

Program EVALPLOT  
(Version 2015-2)

by

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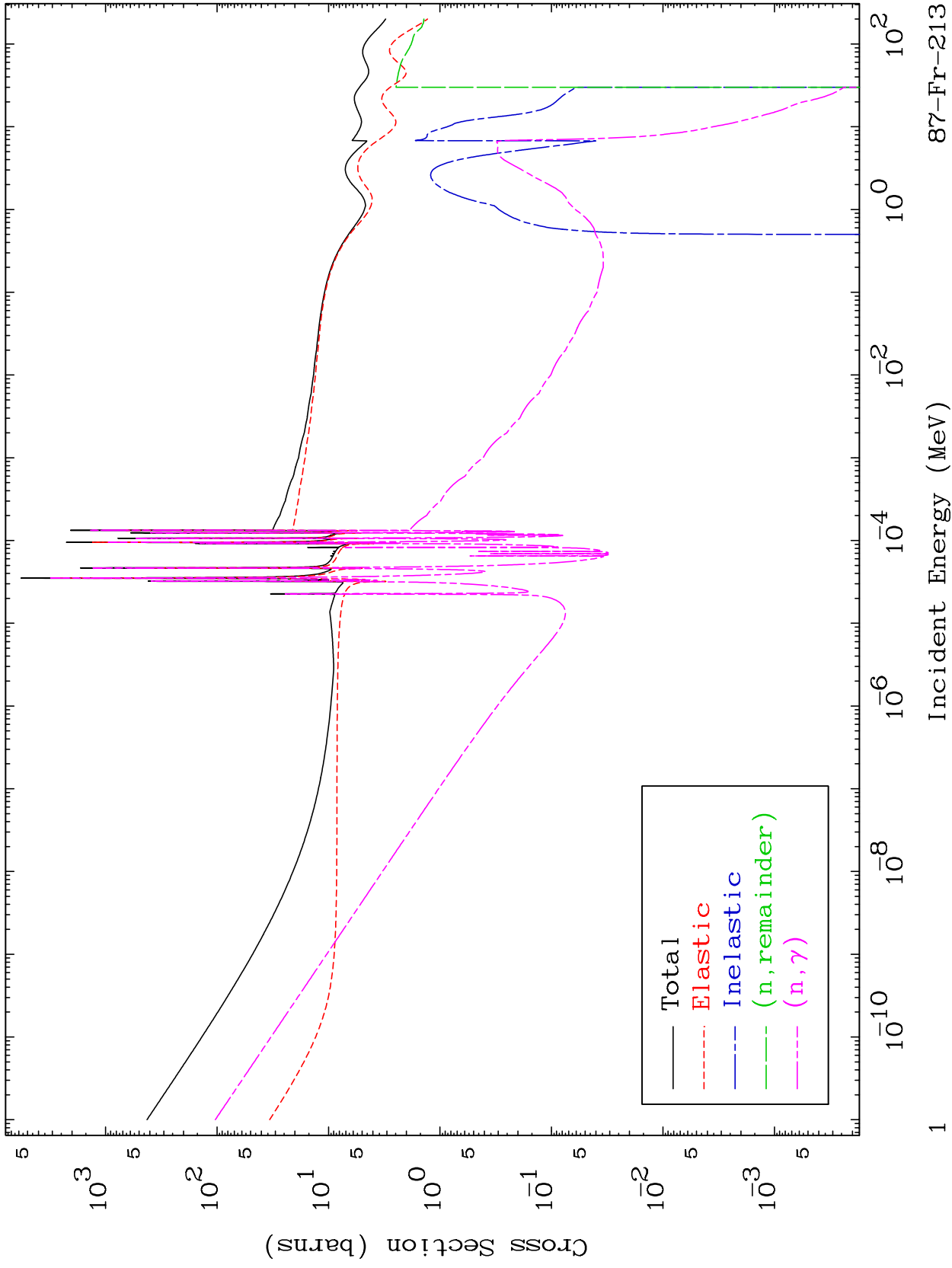
E.Mail:redcullen1@comcast.net  
Web:home.comcast.net/~redcullen1

Press Mouse Button to Start

MAT 8728

Major  
293 Kelvin Cross Sections

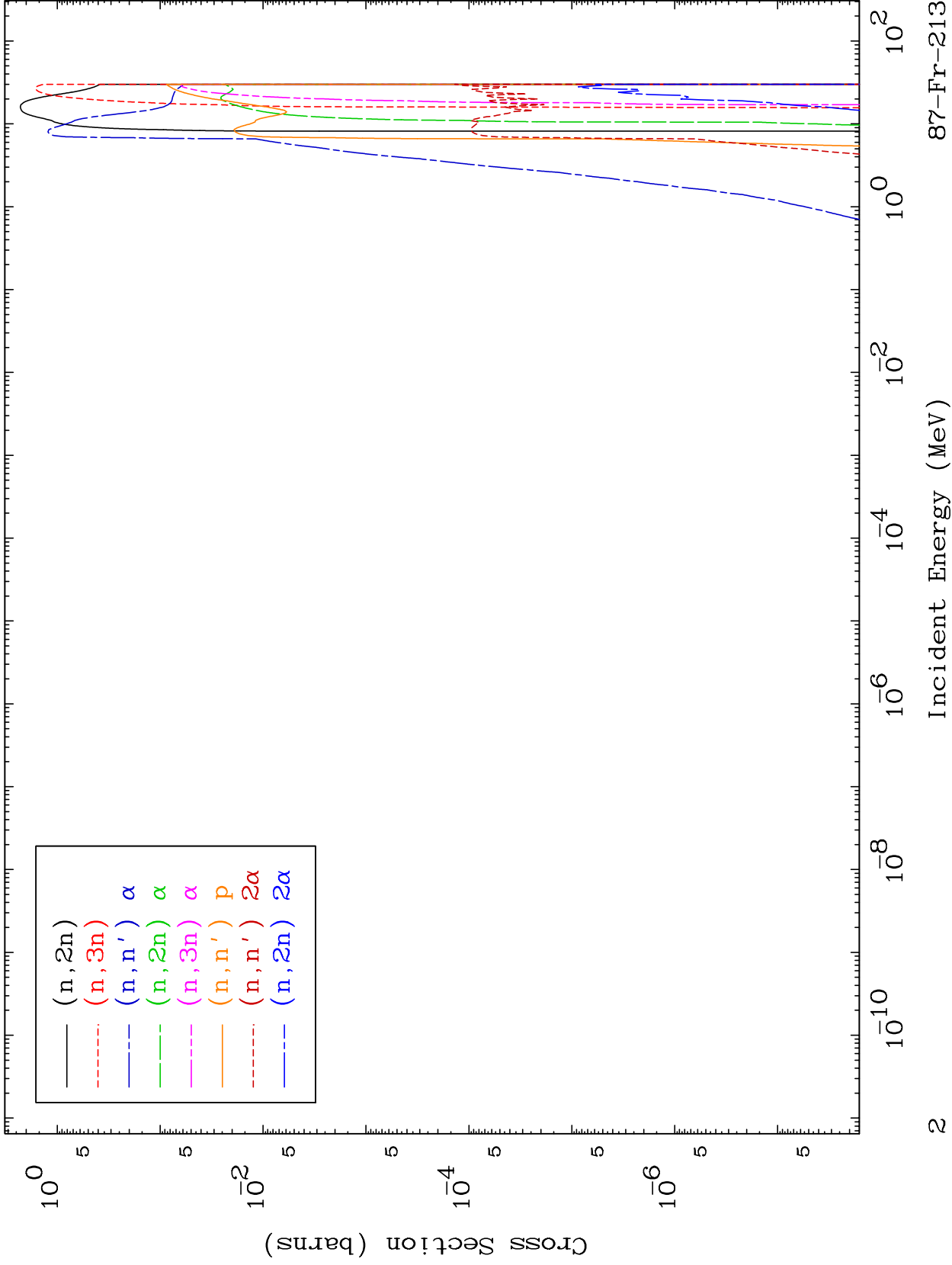
87-Fr-213



MAT 8728

Neutron Production  
293 Kelvin Cross Sections

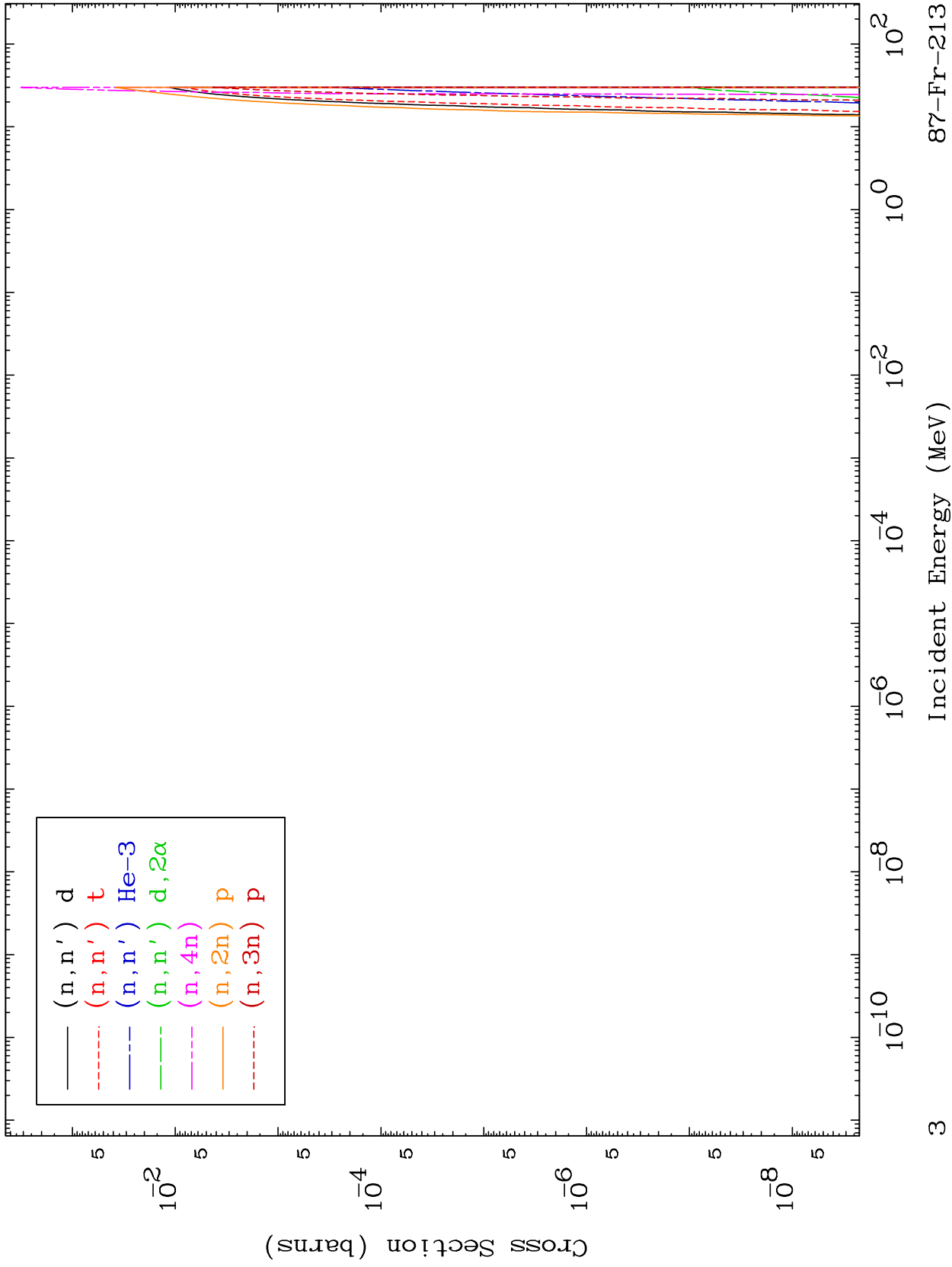
87-Fr-213



MAT 8728

Neutron Production  
293 Kelvin Cross Sections

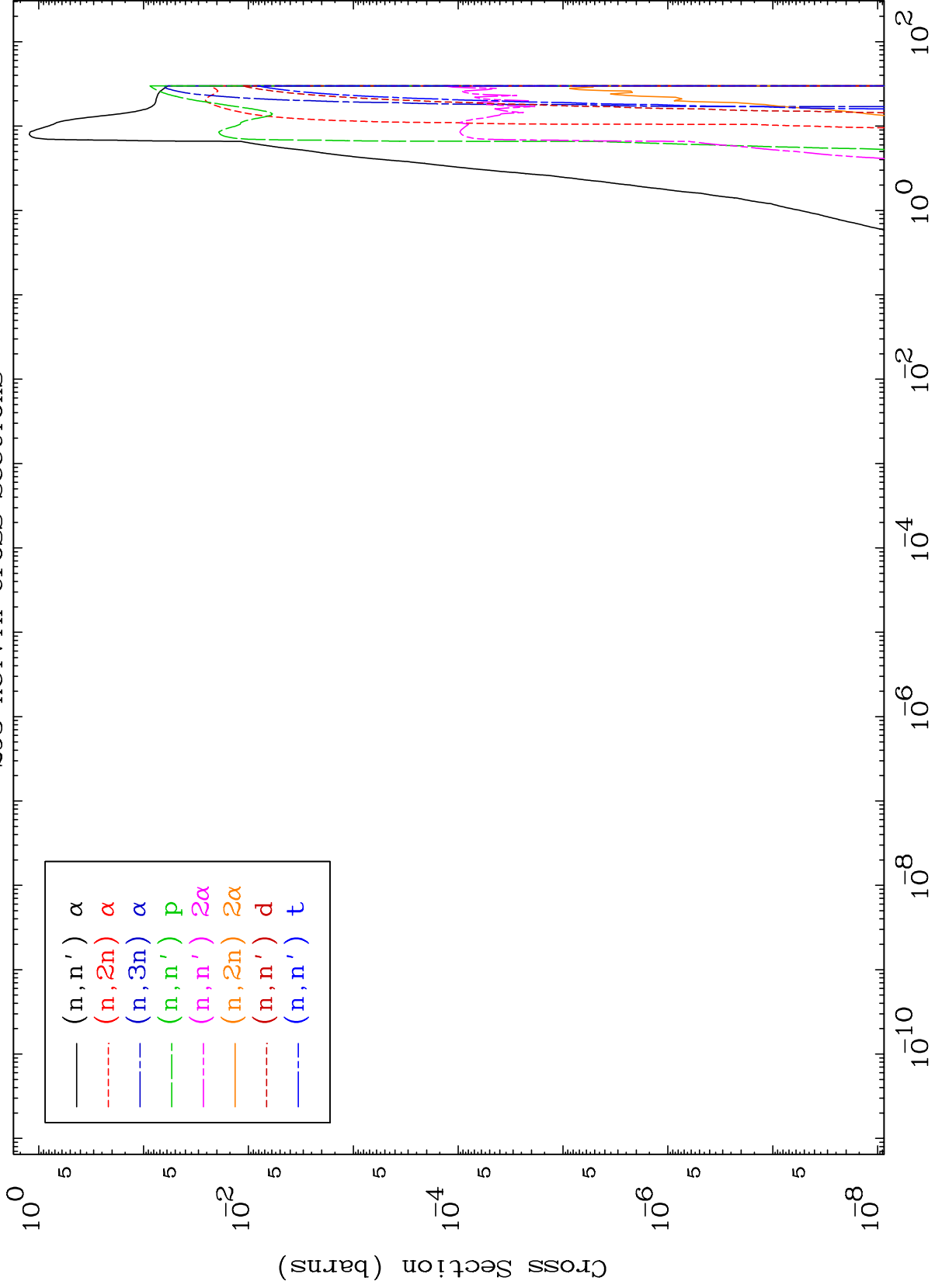
87-Fr-213



MAT 8728

Charged Particle  
293 Kelvin Cross Sections

87-Fr-213



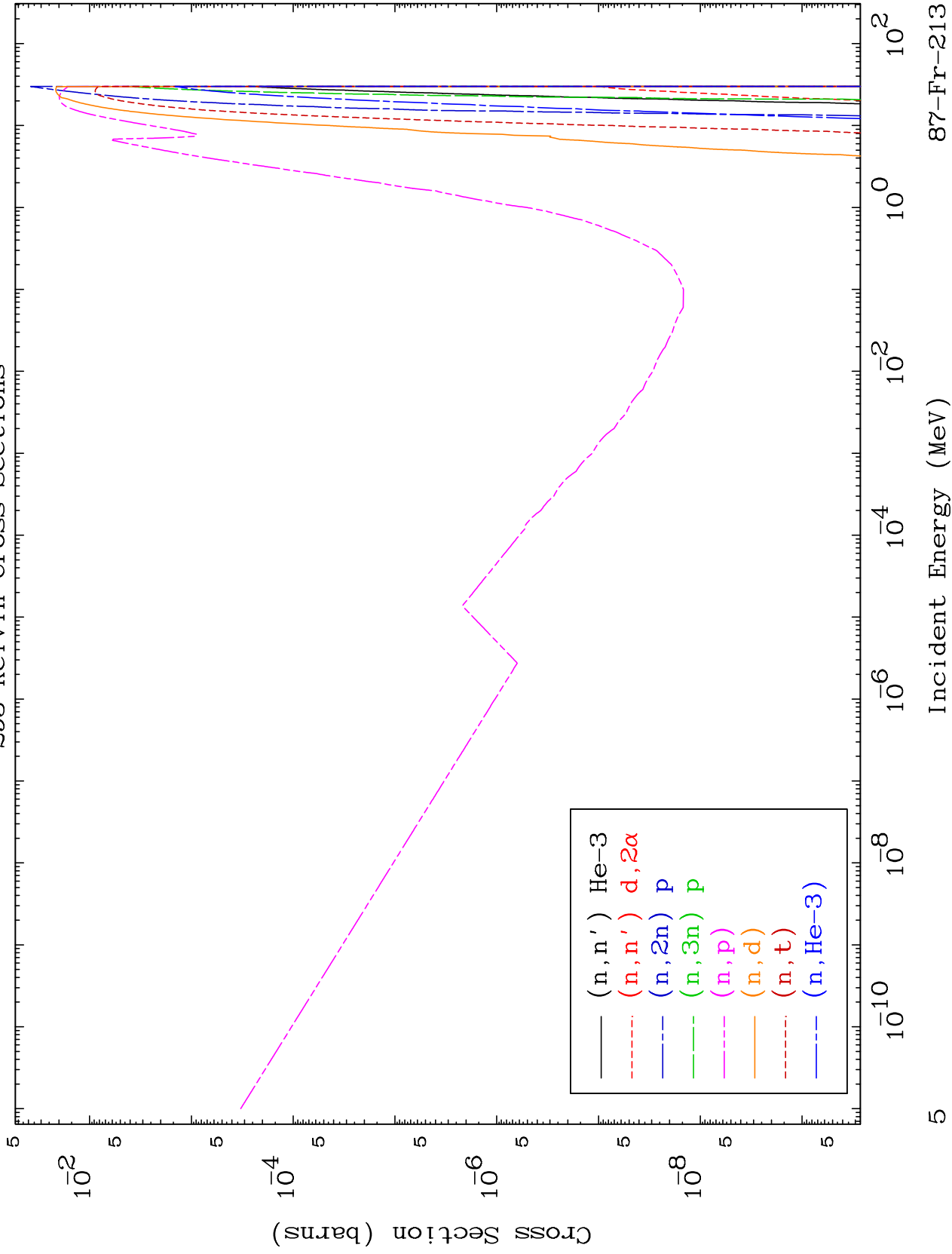
87-Fr-213

Incident Energy (MeV)

MAT 8728

Charged Particle  
293 Kelvin Cross Sections

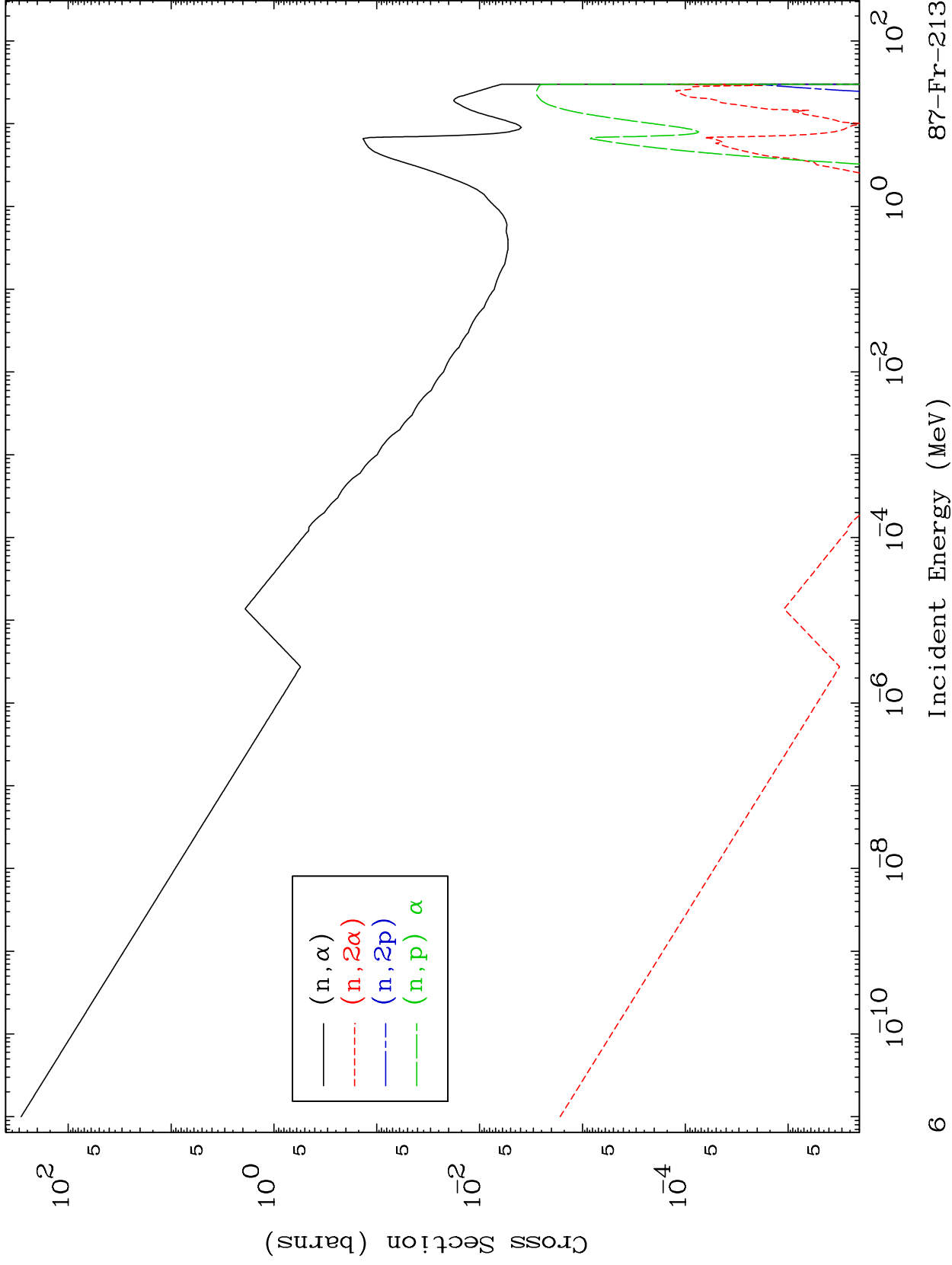
87-Fr-213



MAT 8728

Charged Particle  
293 Kelvin Cross Sections

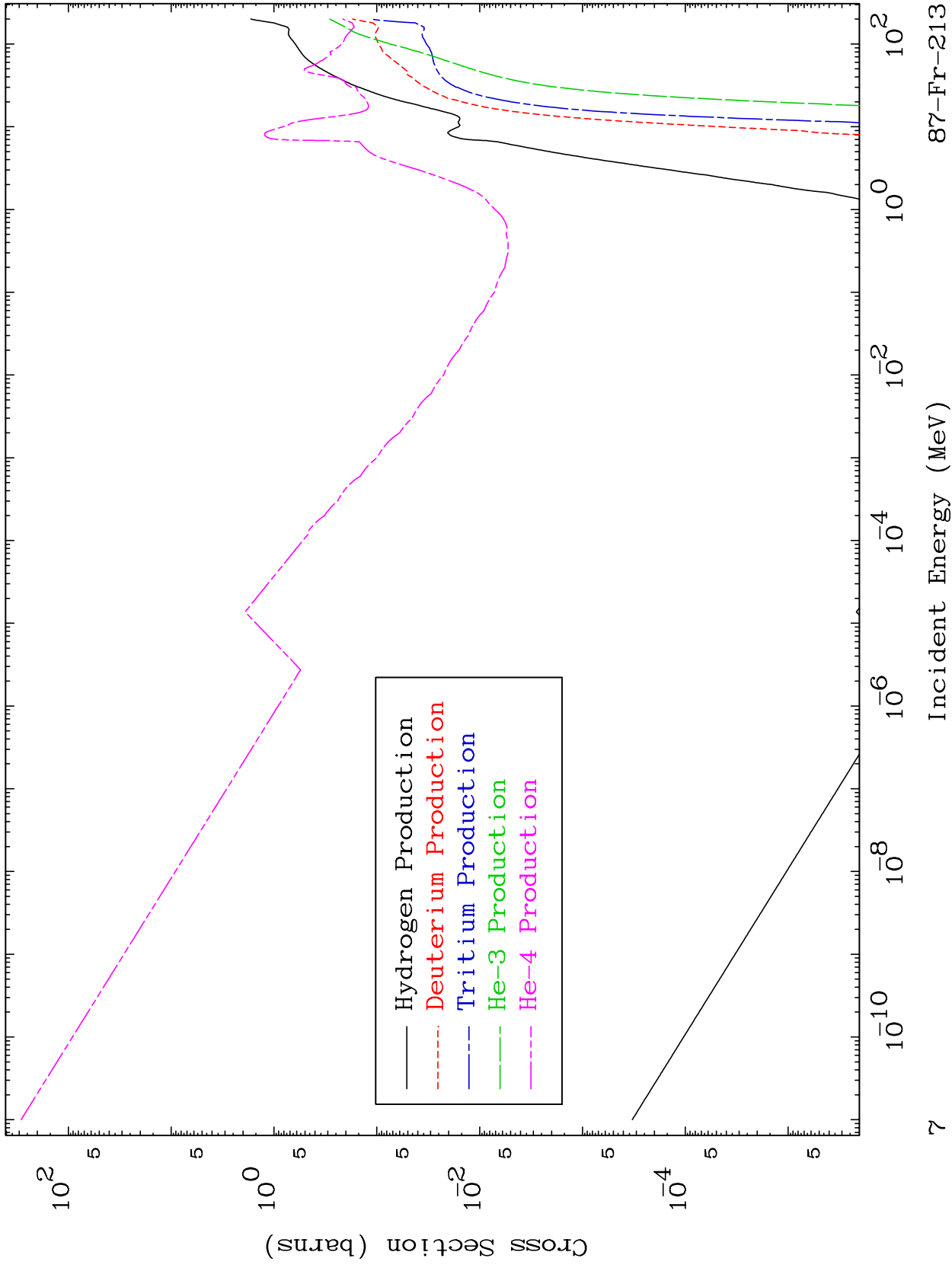
87-Fr-213



MAT 8728

Particle Production  
293 Kelvin Cross Sections

87-Fr-213

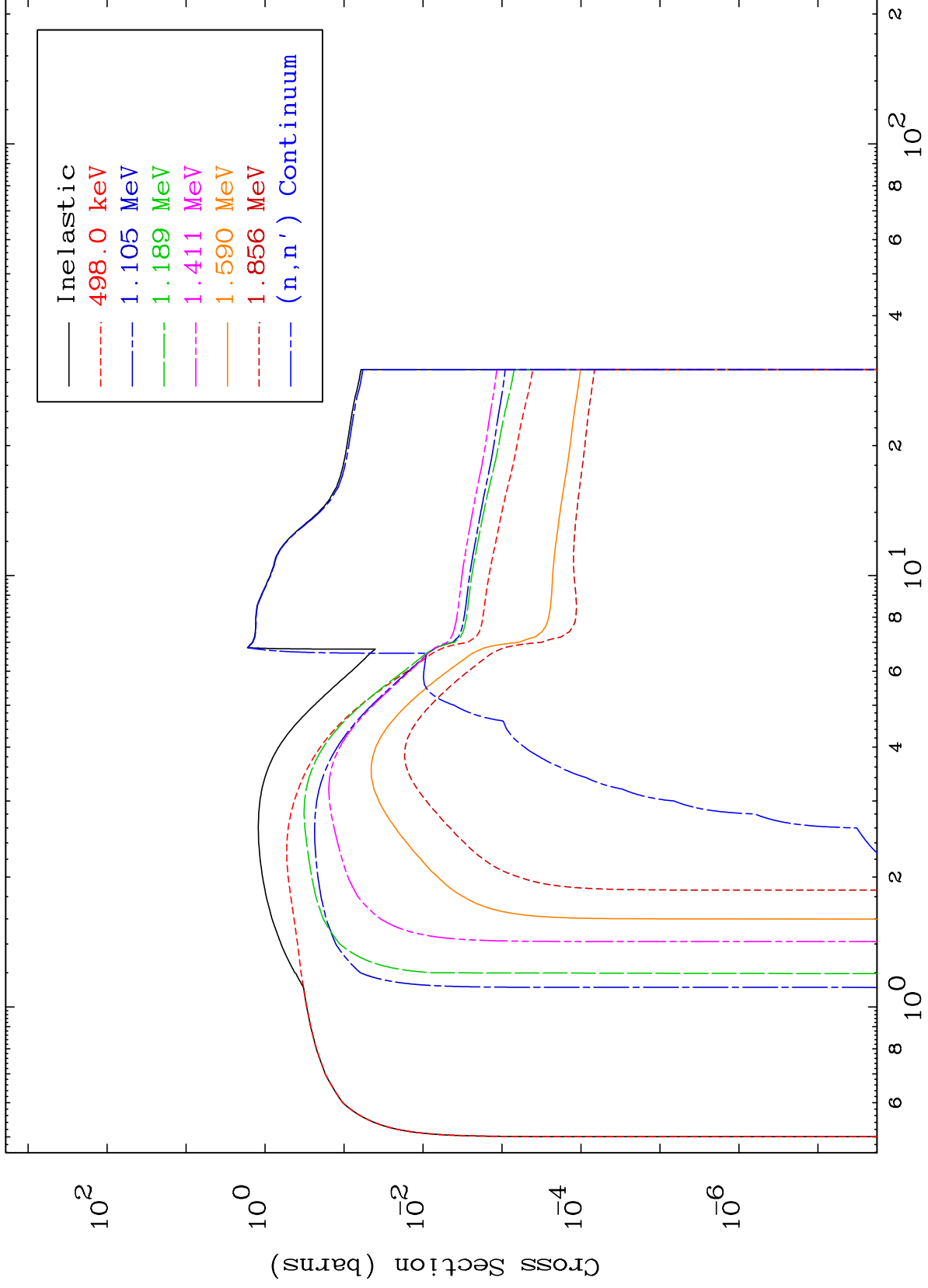




MAT 8728

(n,n') Level  
293 Kelvin Cross Sections

87-Fr-213



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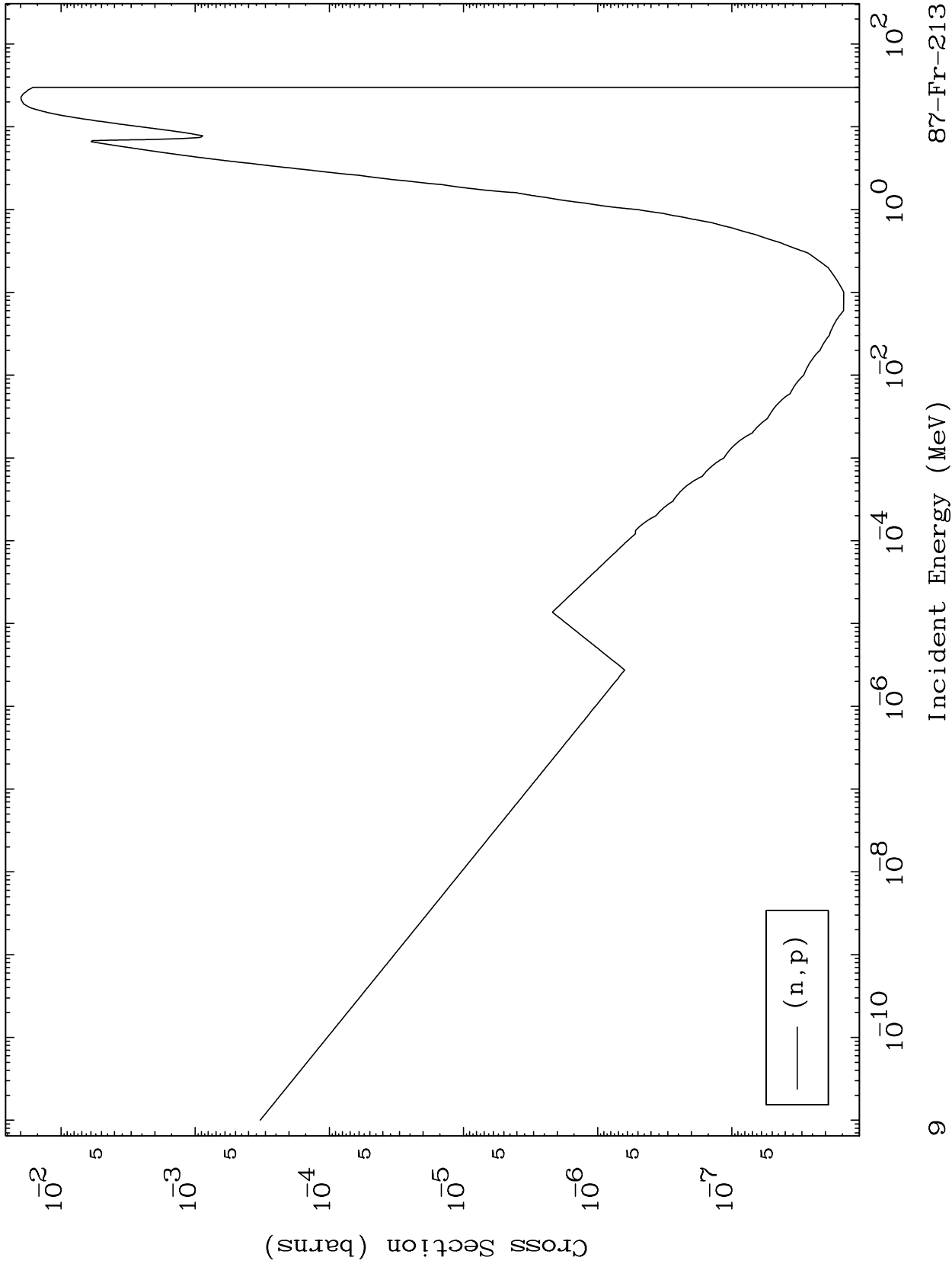
Incident Energy (MeV)

87-Fr-213

MAT 8728

(n,p) Levels  
293 Kelvin Cross Sections

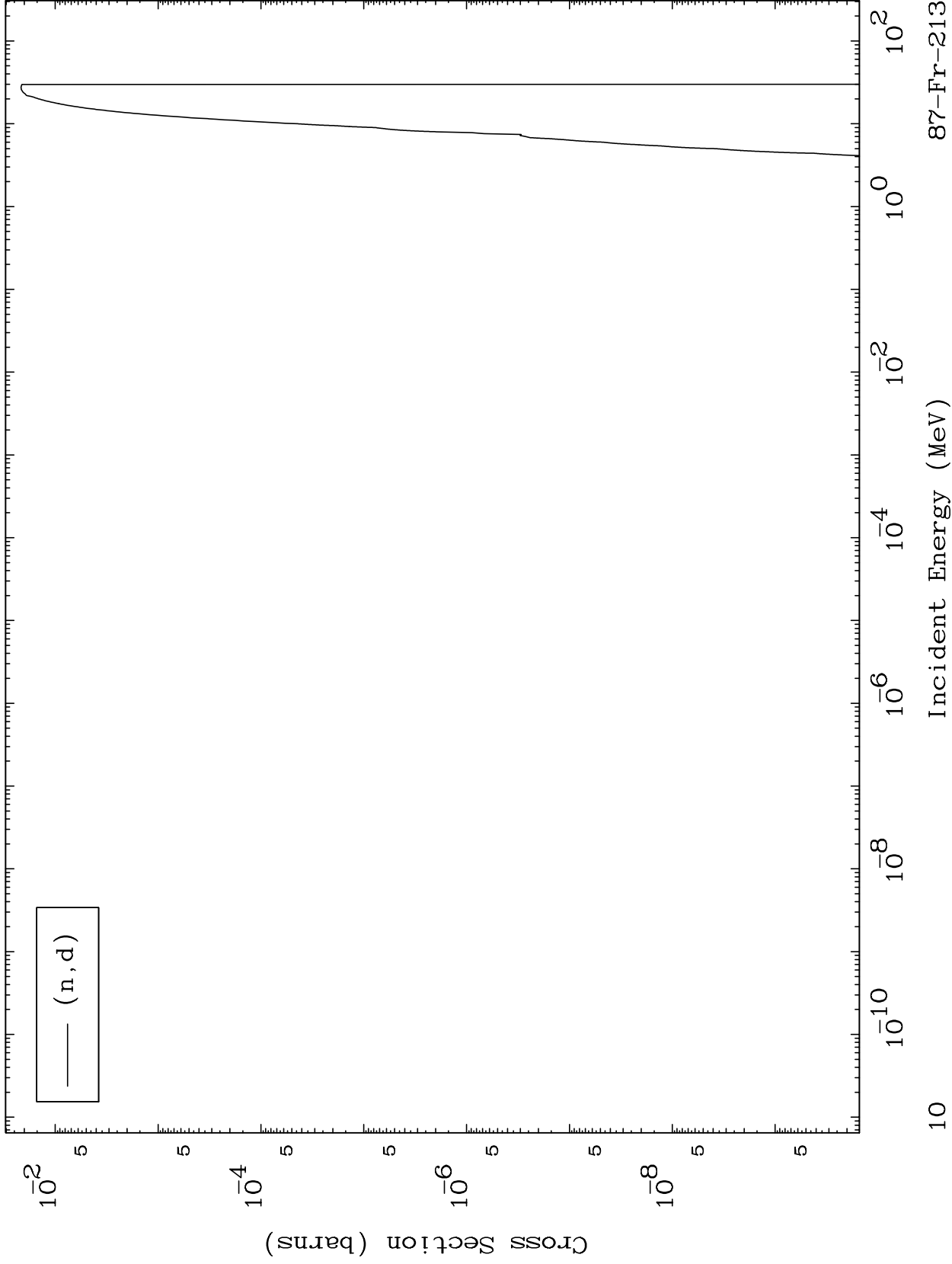
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MAT 8728

(n,d) Levels  
293 Kelvin Cross Sections

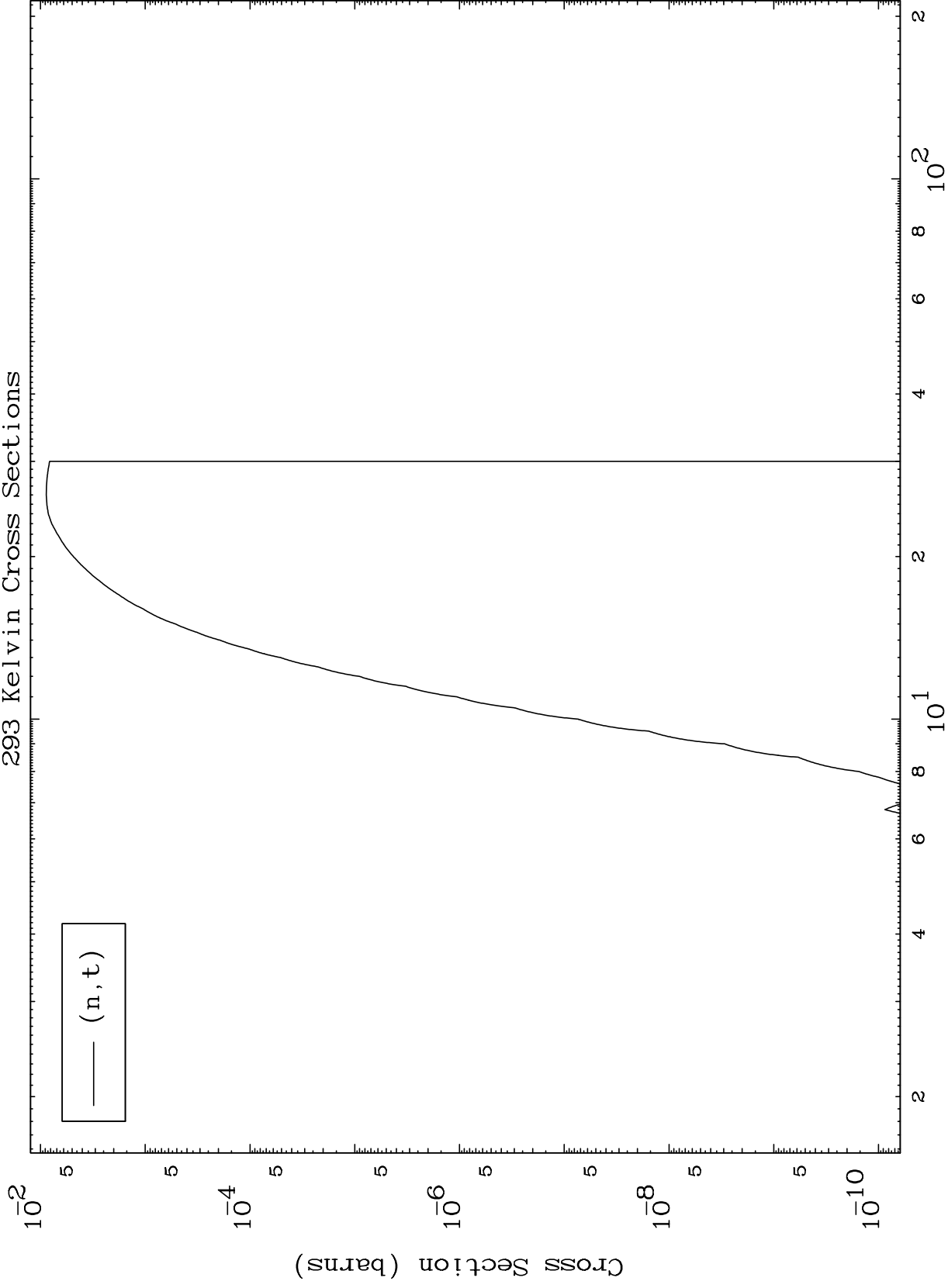
87-Fr-213



MAT 8728

(n,t) Levels  
293 Kelvin Cross Sections

87-Fr-213



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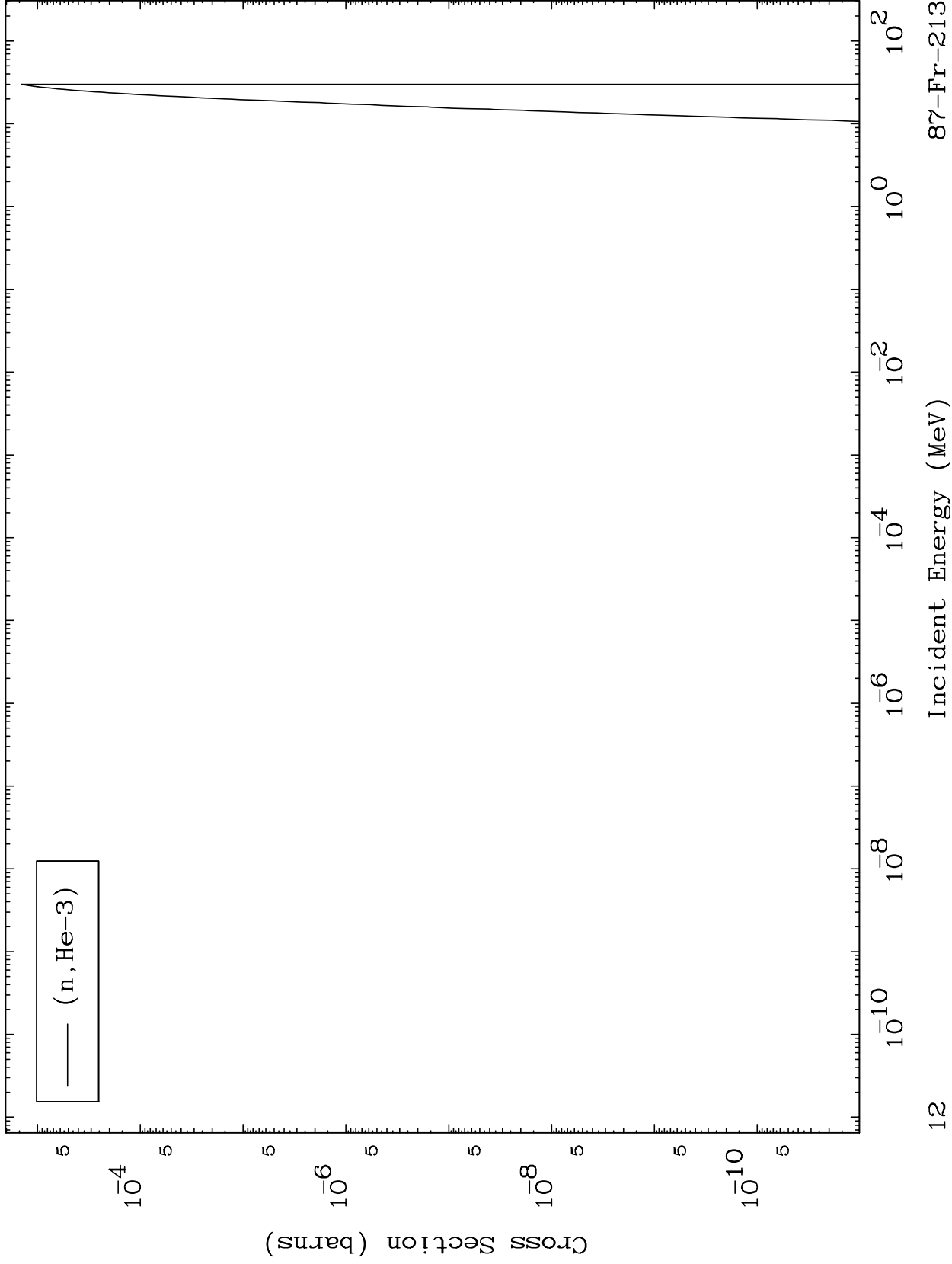
Incident Energy (MeV)

87-Fr-213

MAT 8728

(n,He3) Levels  
293 Kelvin Cross Sections

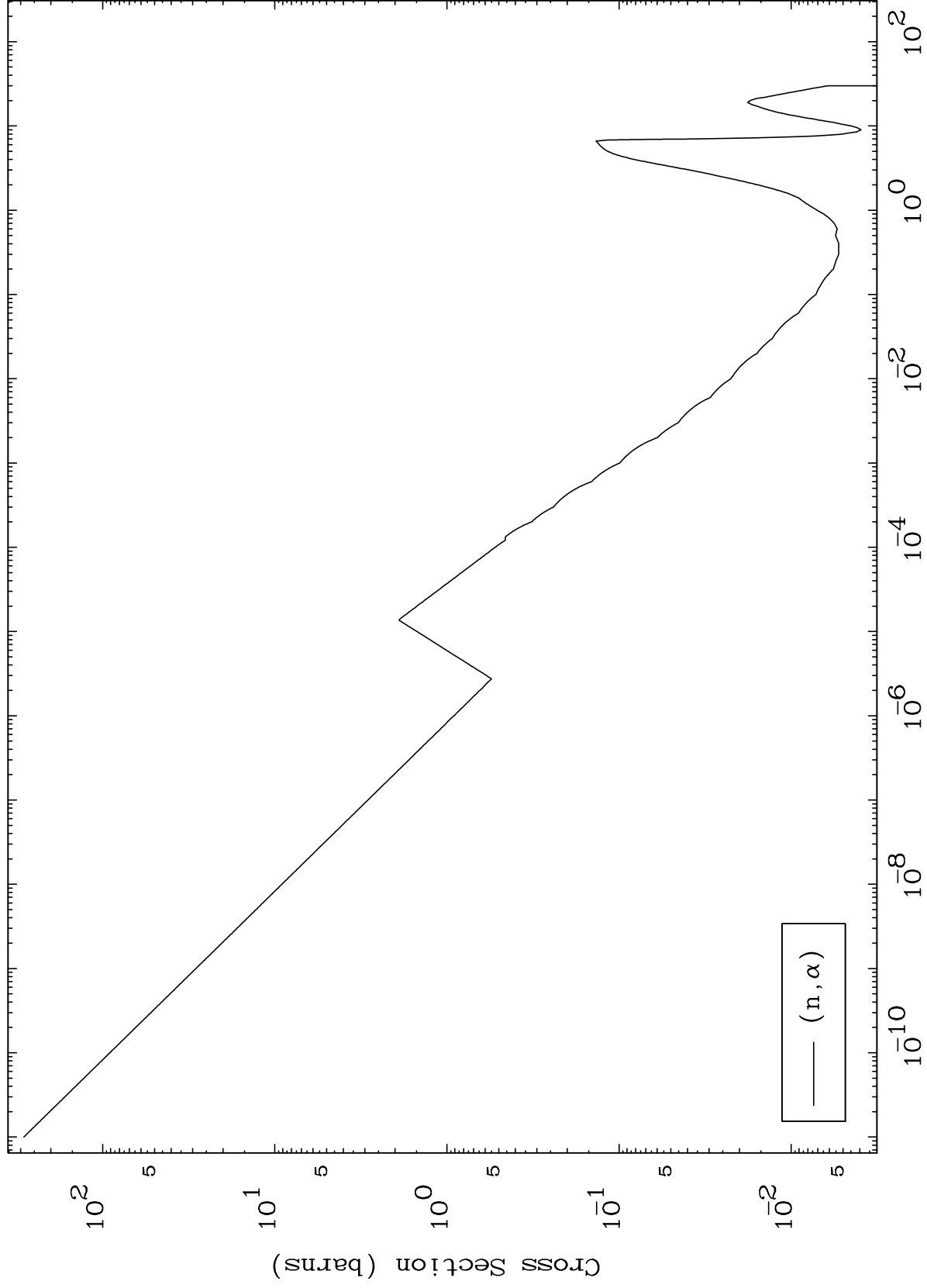
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MAT 8728

(n,  $\alpha$ ) Levels  
293 Kelvin Cross Sections

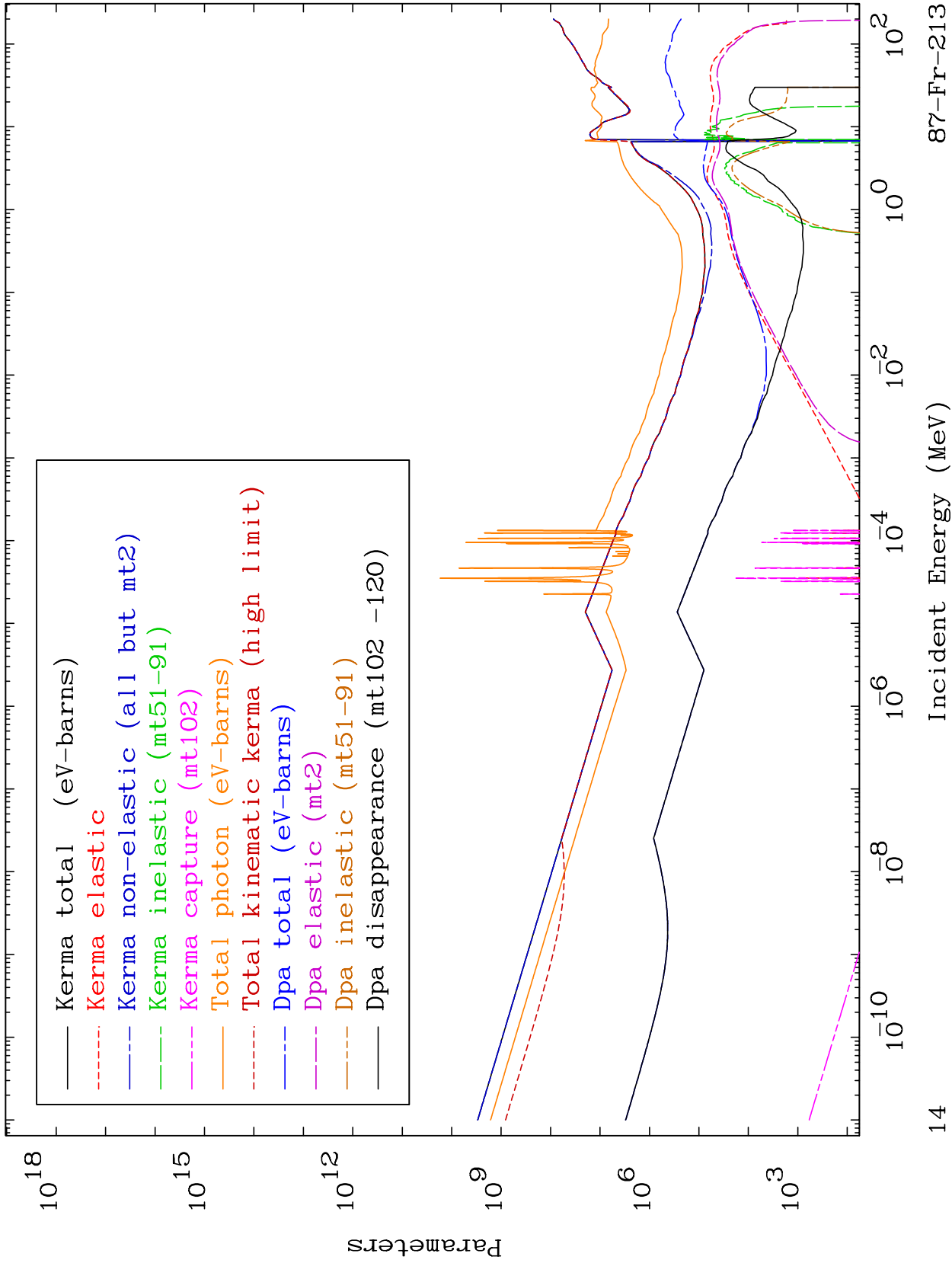
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Incident Energy (MeV)

87-Fr-213

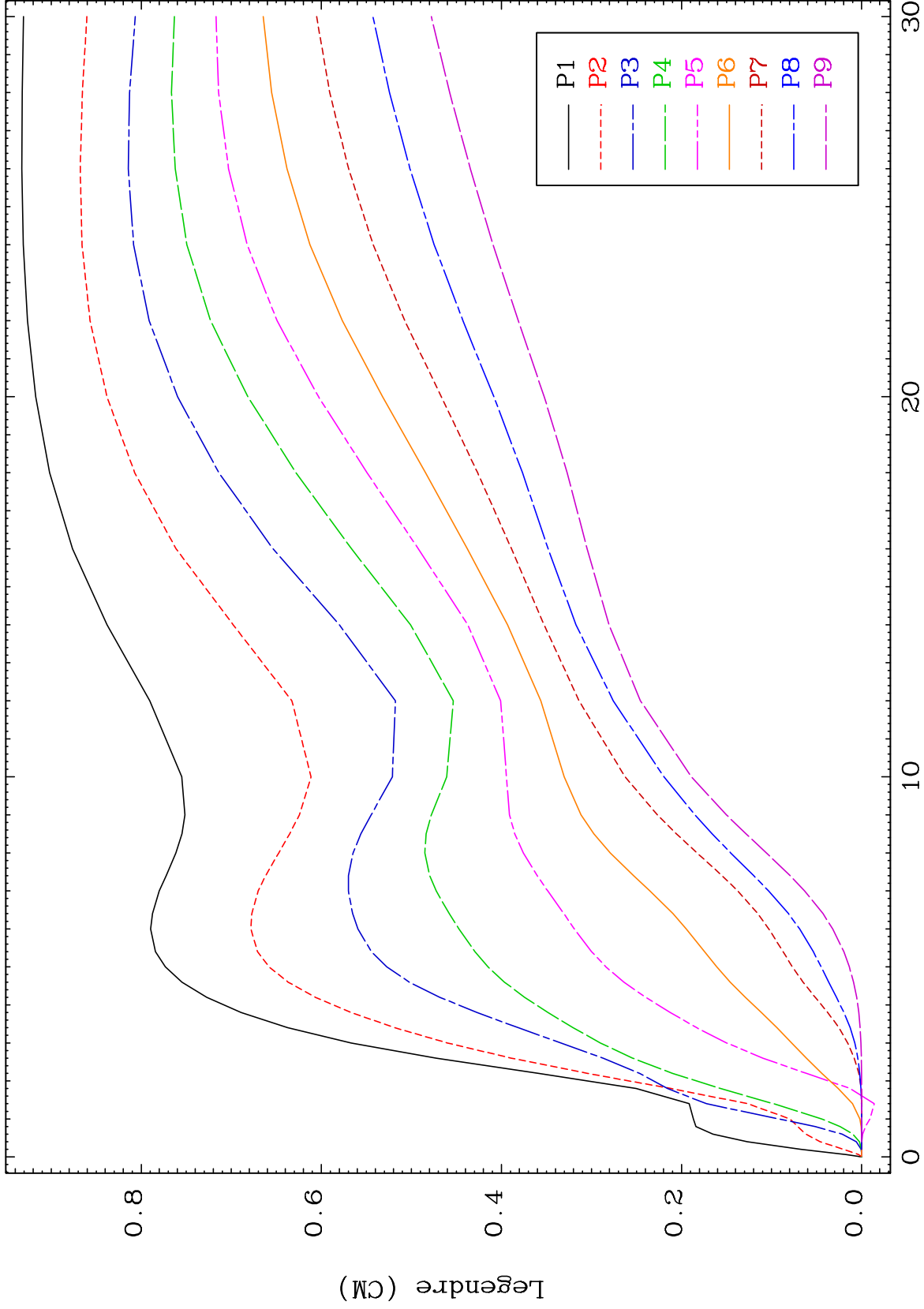
13



MAT 8728

Elastic Legendre Coefficients

87-Fr-213



87-Fr-213

Incident Energy (MeV)

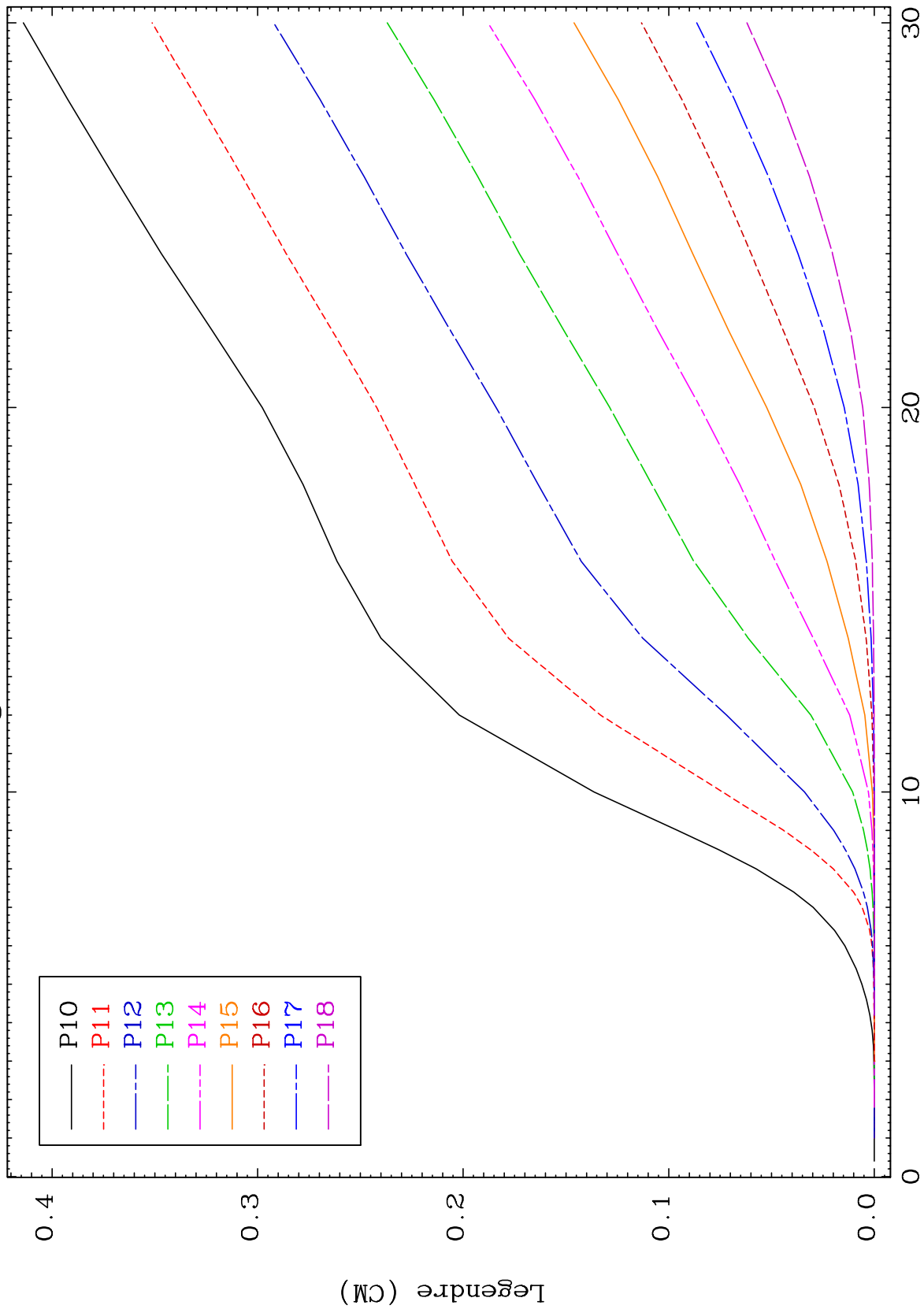
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MAT 8728

Elastic Legendre Coefficients

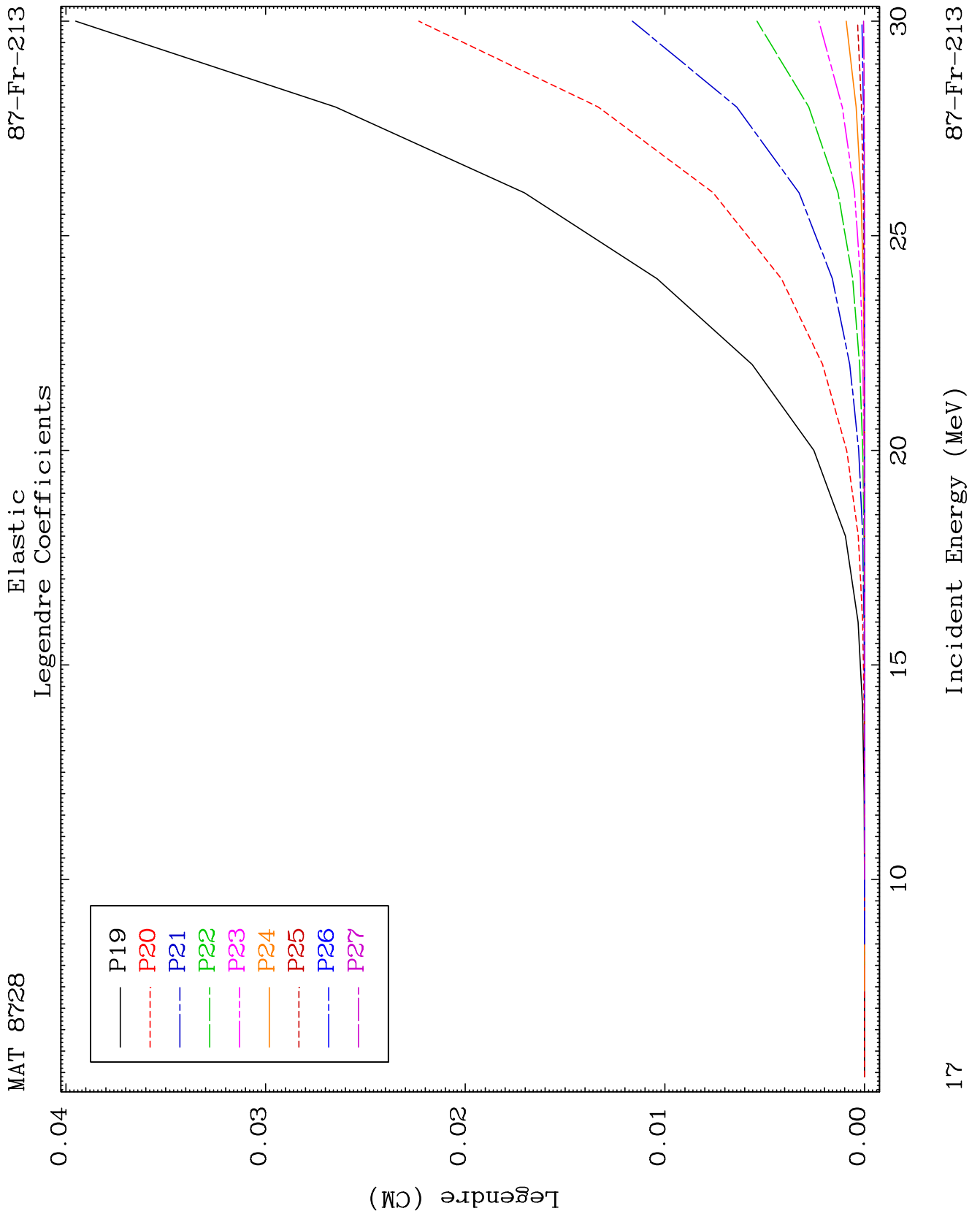
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16

Incident Energy (MeV)

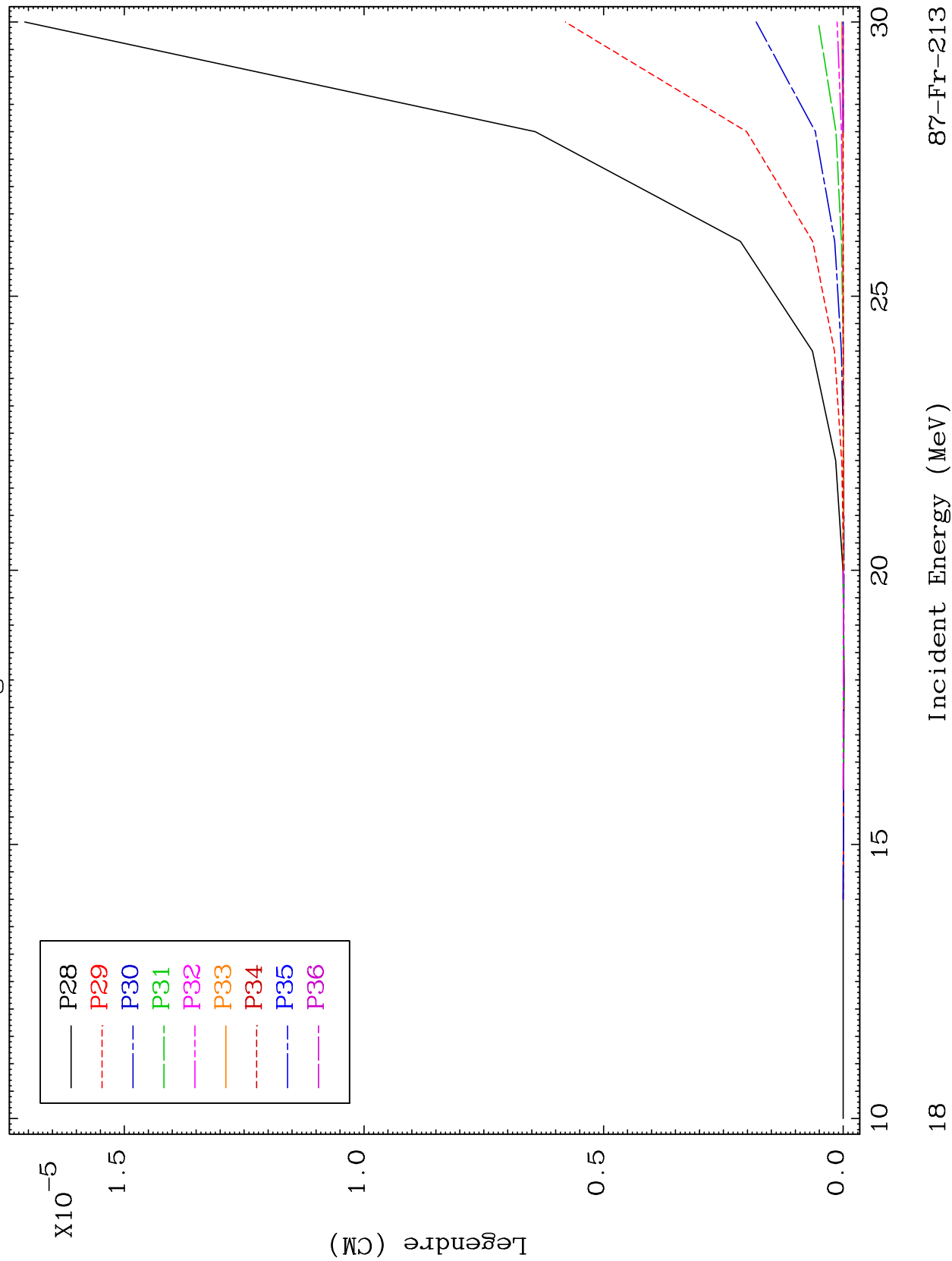
87-Fr-213



MAT 8728

Elastic Legendre Coefficients

87-Fr-213



87-Fr-213

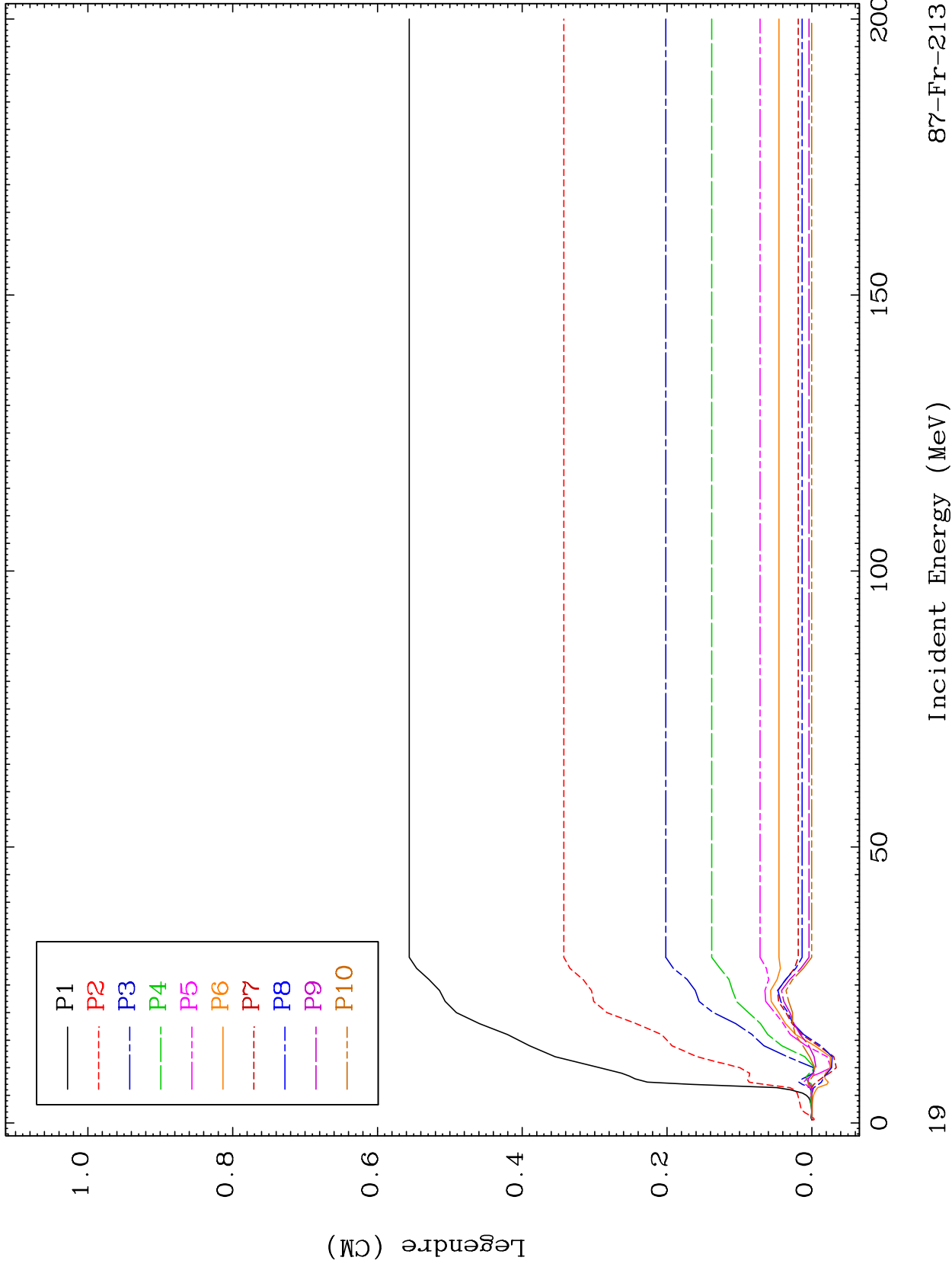
Incident Energy (MeV)

18

MAT 8728

498.0 keV (n, n') Level  
Legendre Coefficients

87-Fr-213



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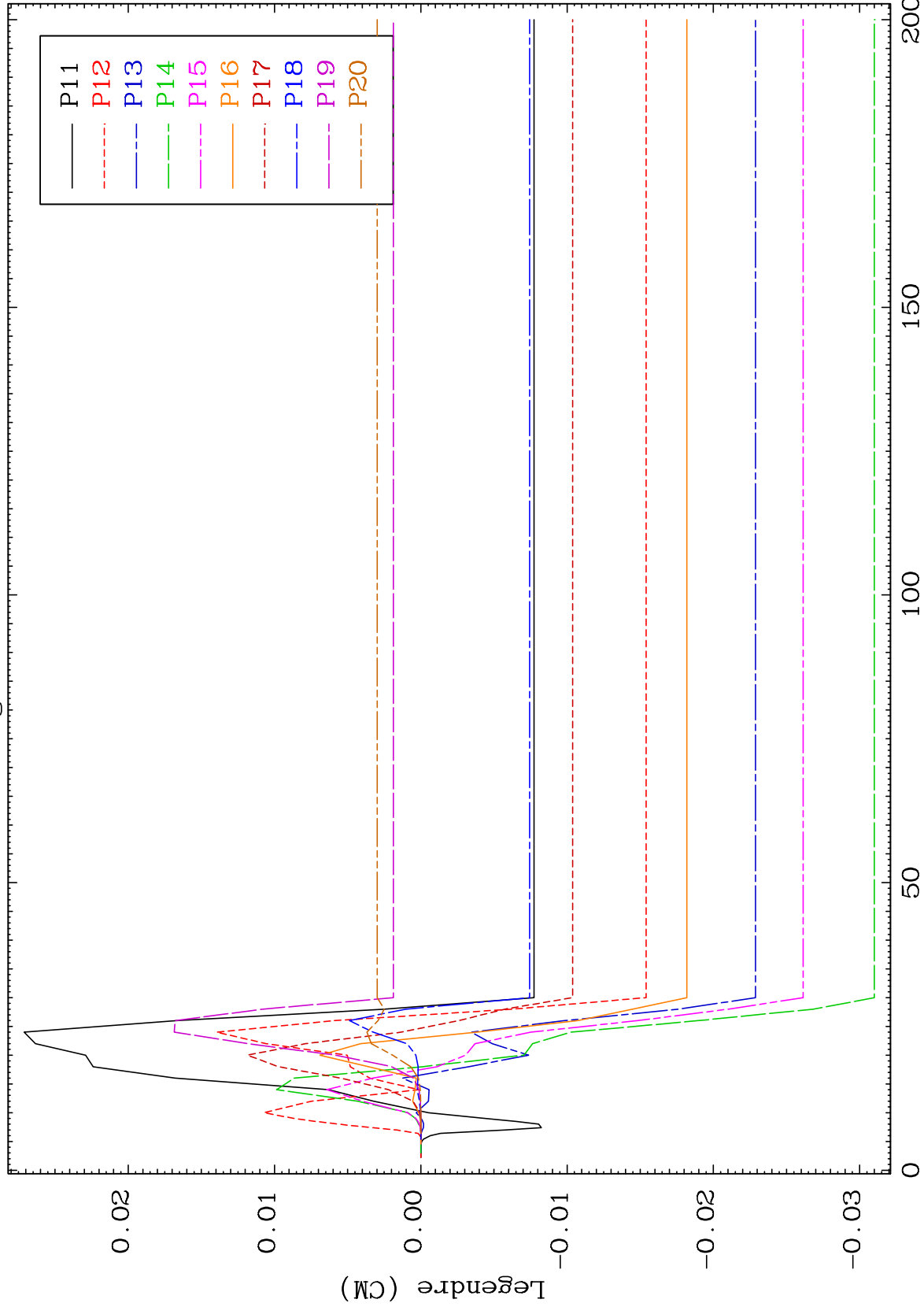
Incident Energy (MeV)

87-Fr-213

MAT 8728

498.0 keV (n, n') Level  
Legendre Coefficients

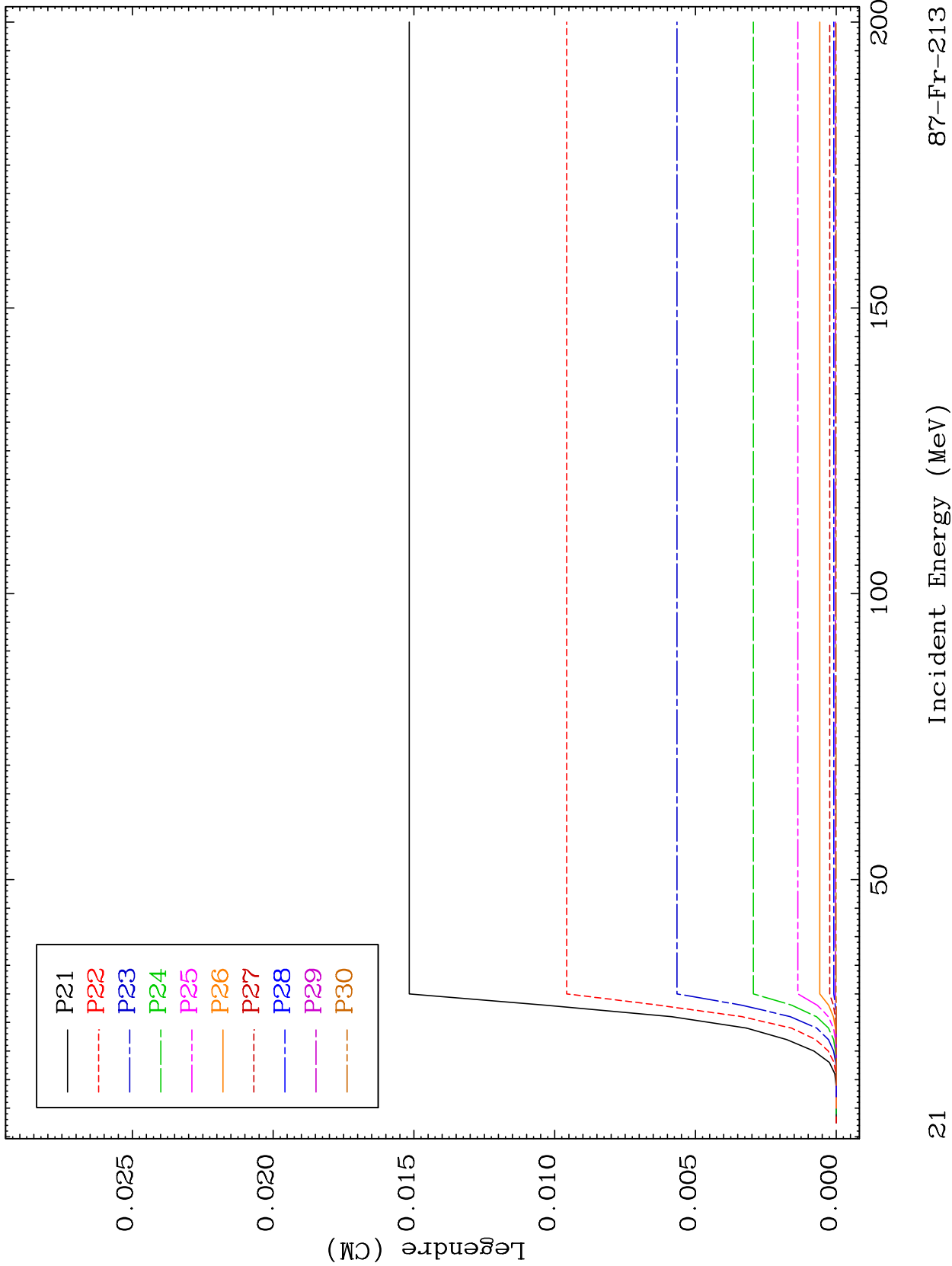
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87-Fr-213

Incident Energy (MeV)

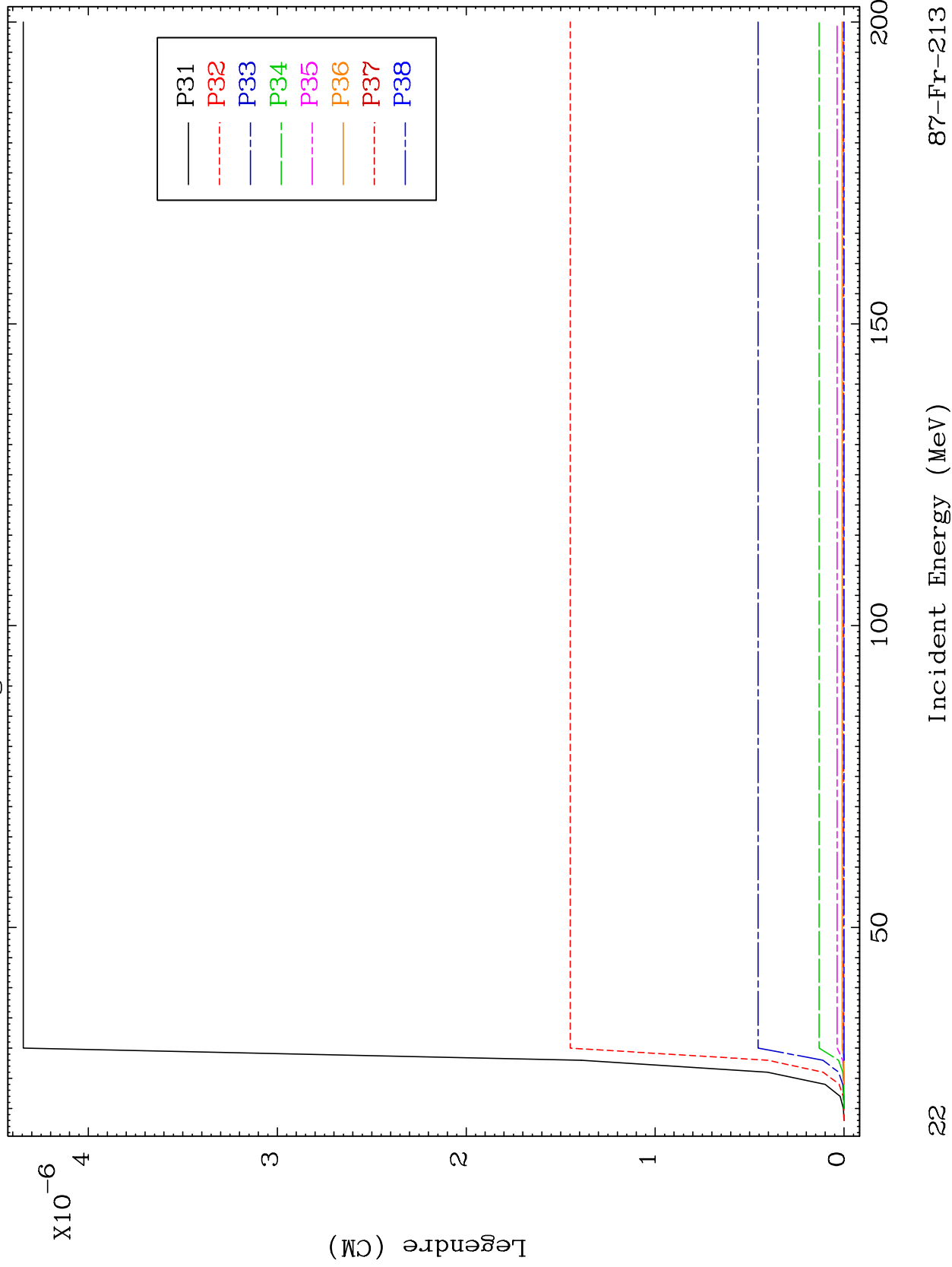
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MAT 8728

498.0 keV (n,n') Level  
Legendre Coefficients

87-Fr-213

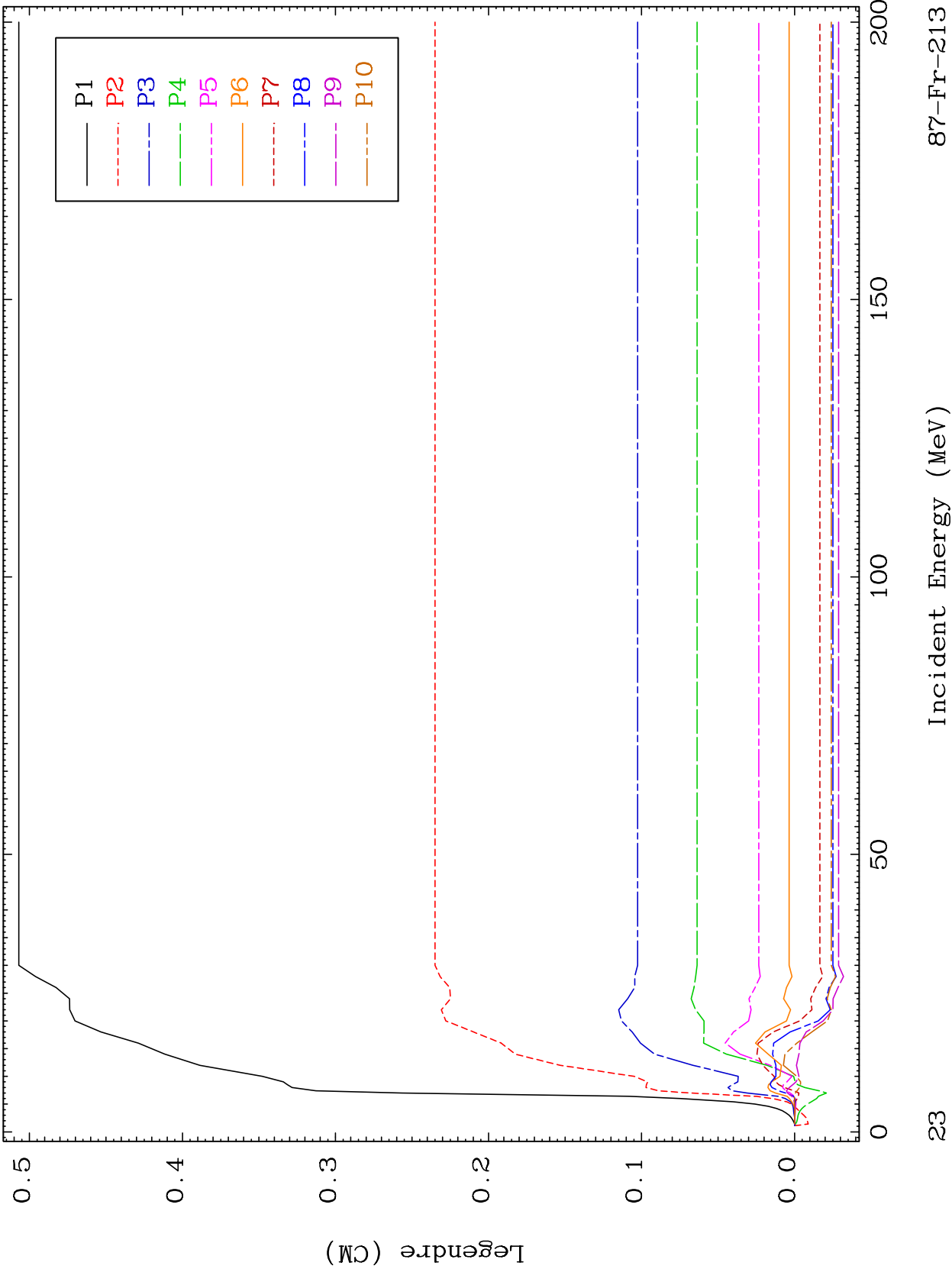


22

MAT 8728

1.105 MeV (n,n') Level  
Legendre Coefficients

87-Fr-213



23

Incident Energy (MeV)

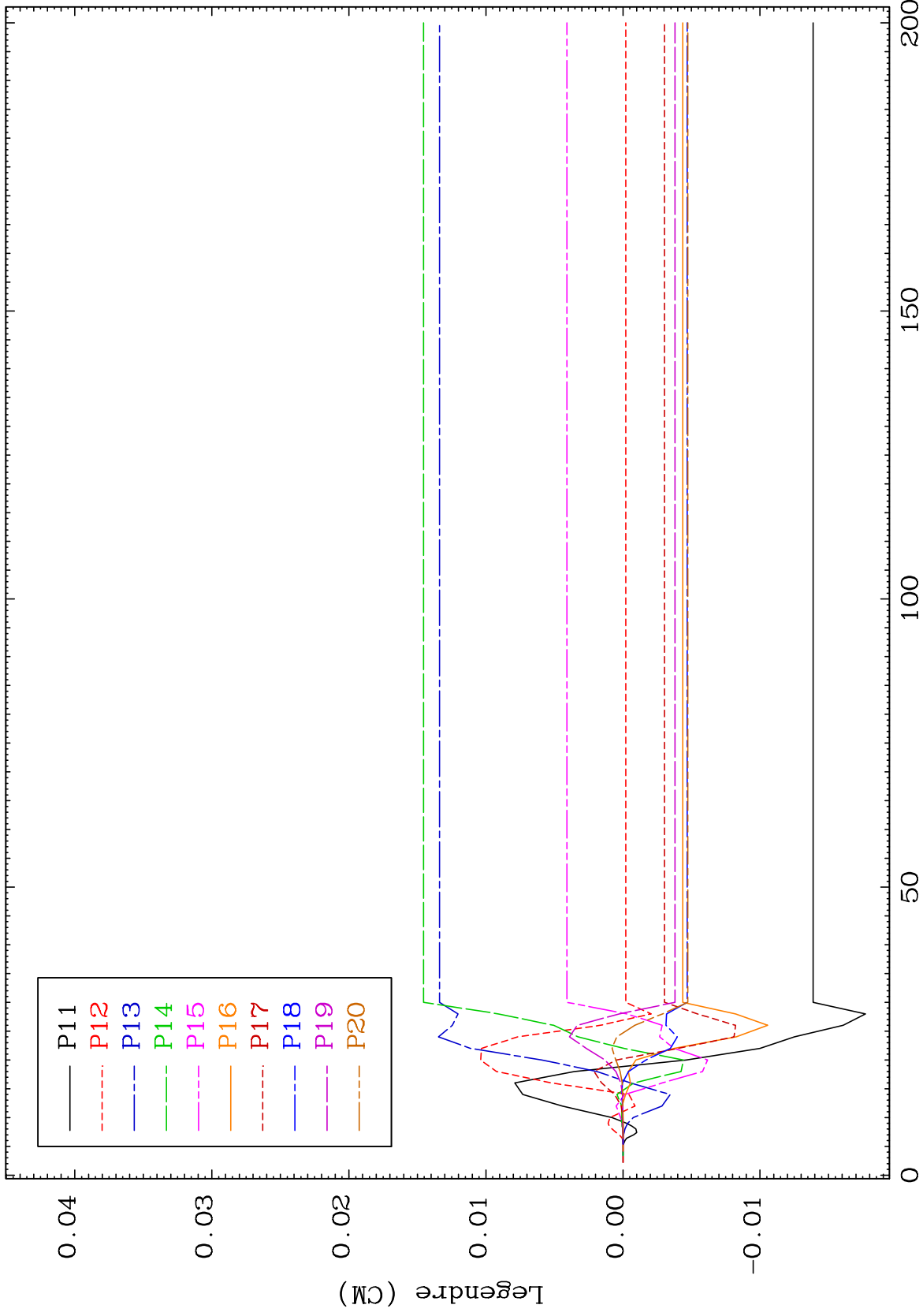
87-Fr-213



MAT 8728

1.105 MeV (n, n') Level  
Legendre Coefficients

87-Fr-213



24

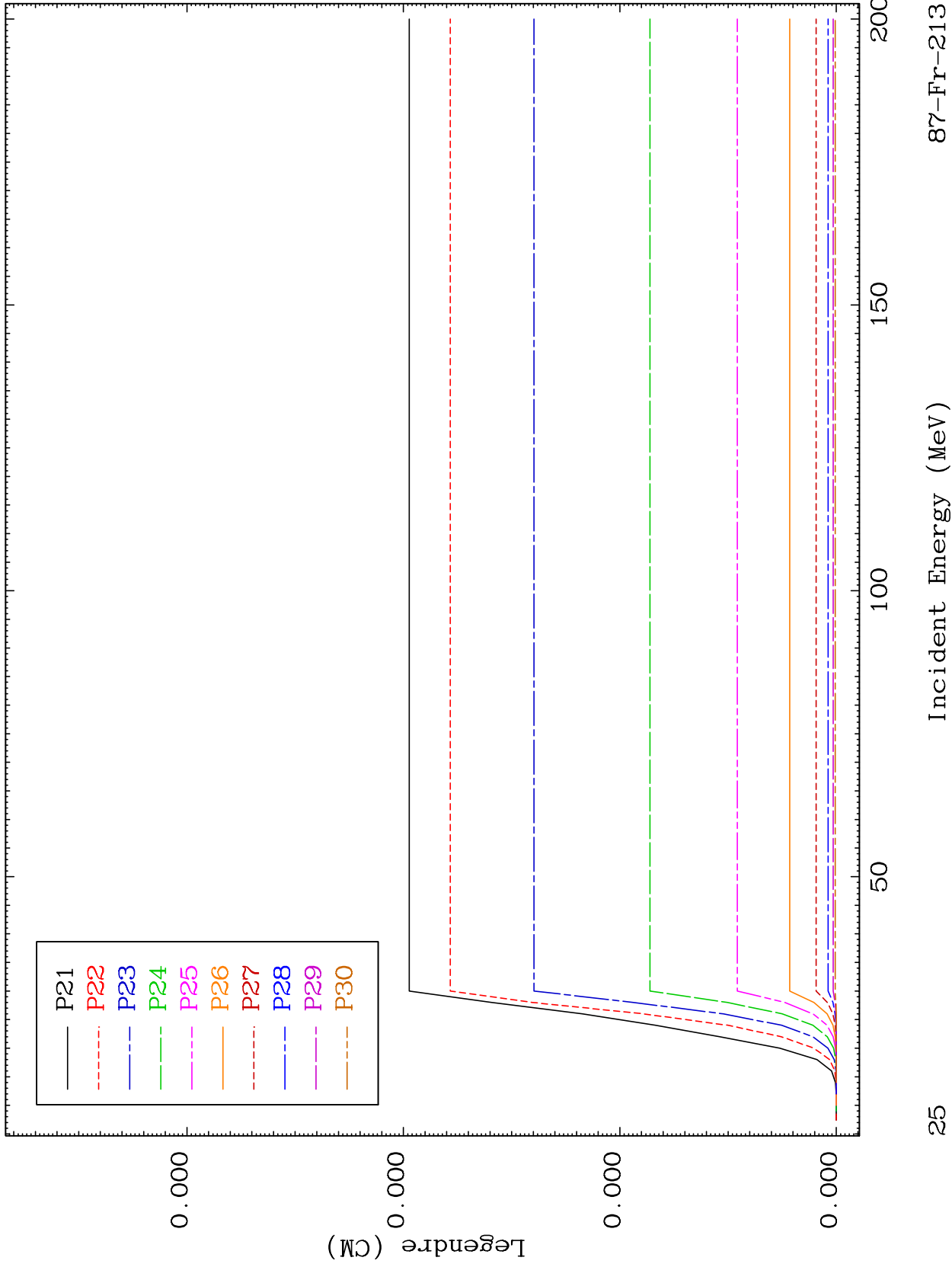
Incident Energy (MeV)

87-Fr-213

MAT 8728

1.105 MeV (n,n') Level  
Legendre Coefficients

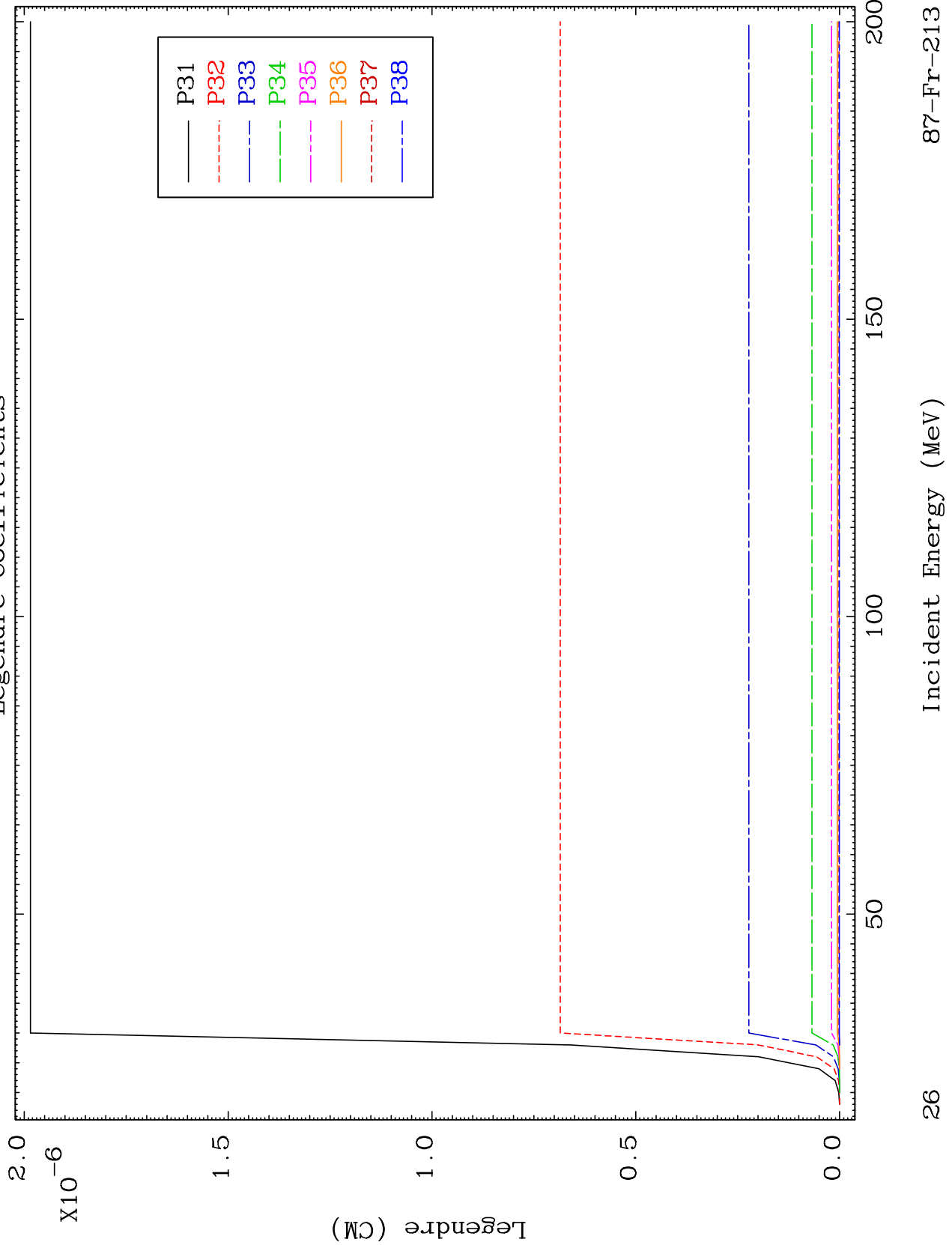
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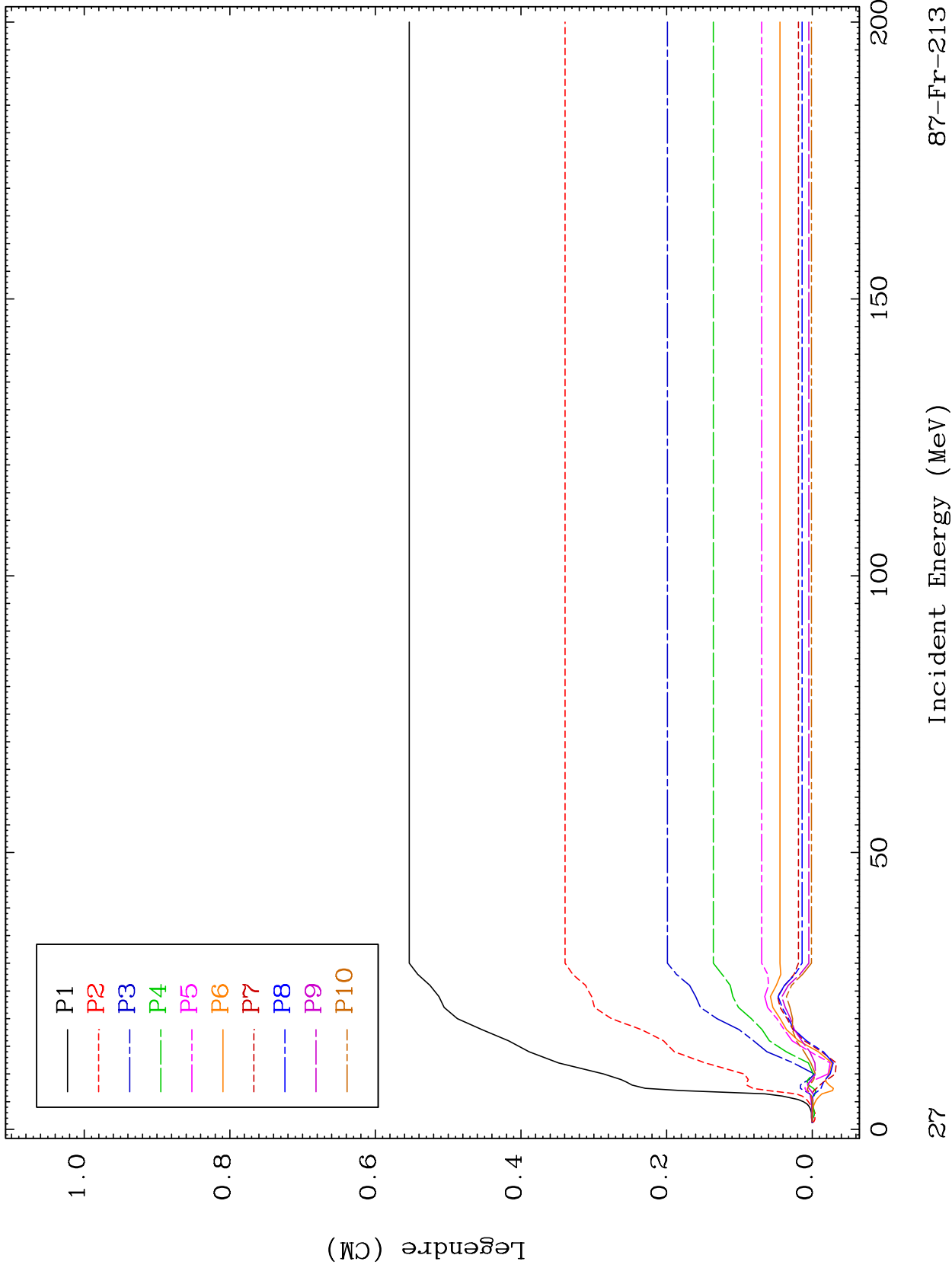


25

Incident Energy (MeV)

87-Fr-213

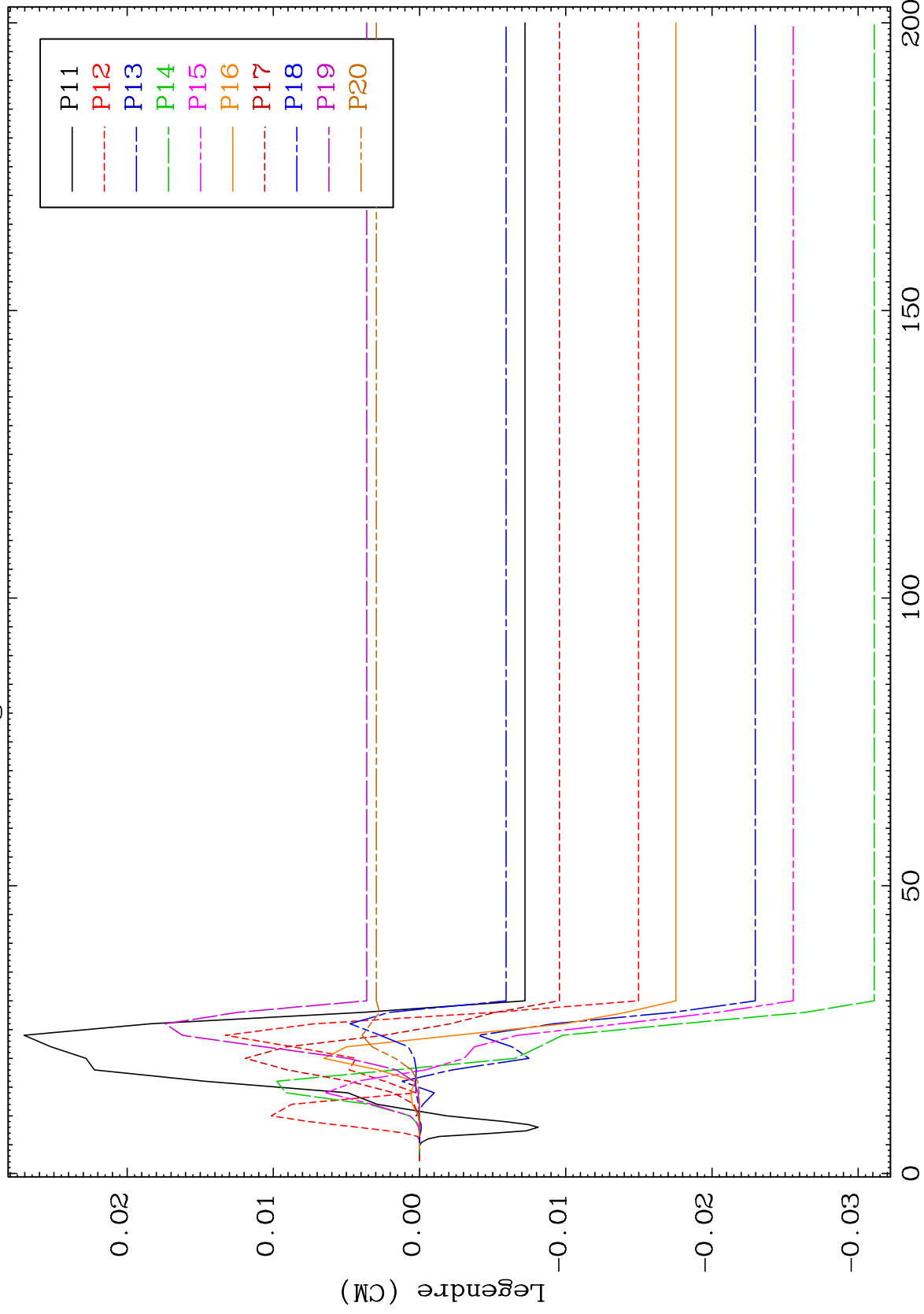




MAT 8728

1.189 MeV (n,n') Level  
Legendre Coefficients

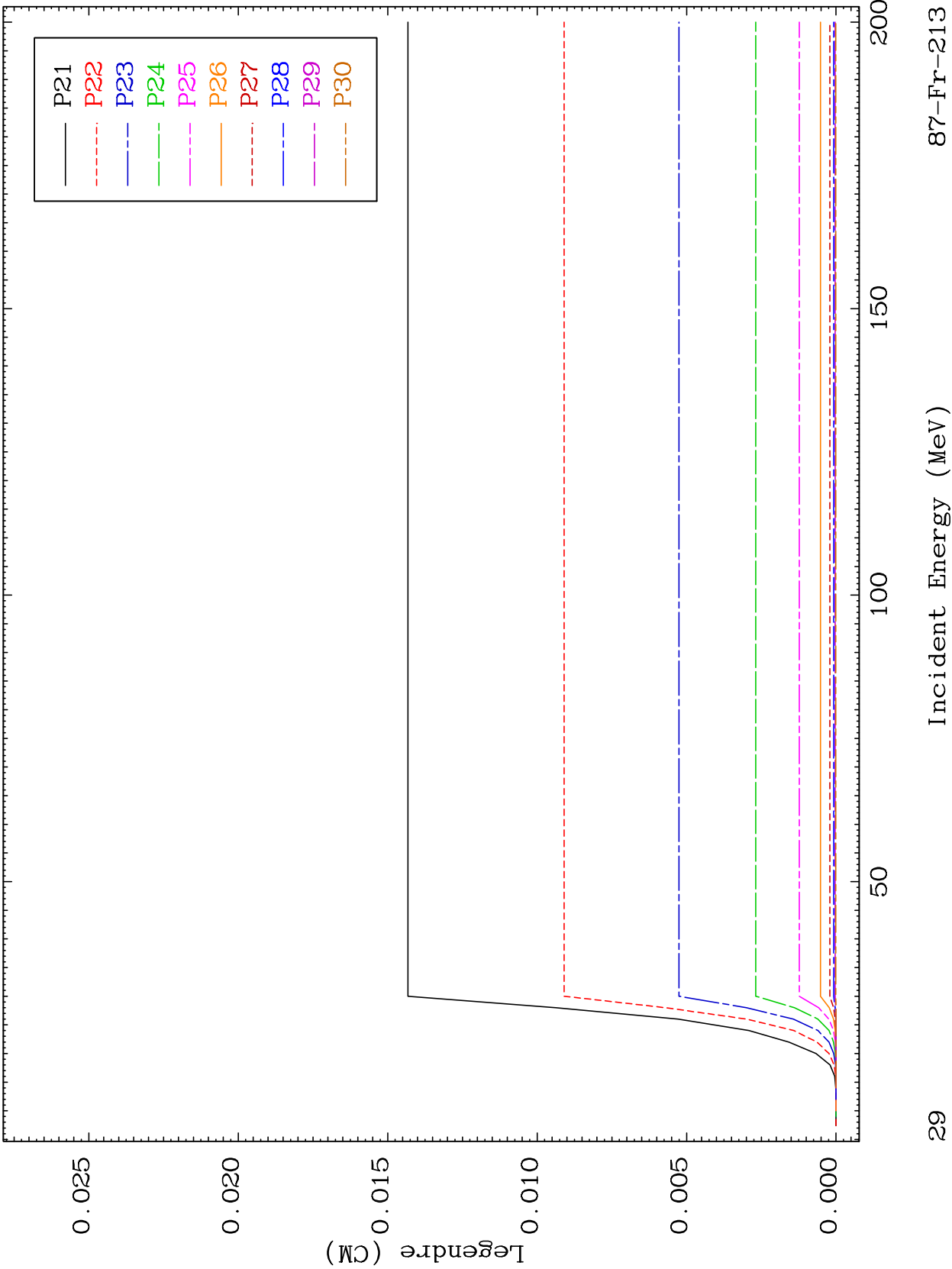
87-Fr-213



28

Incident Energy (MeV)

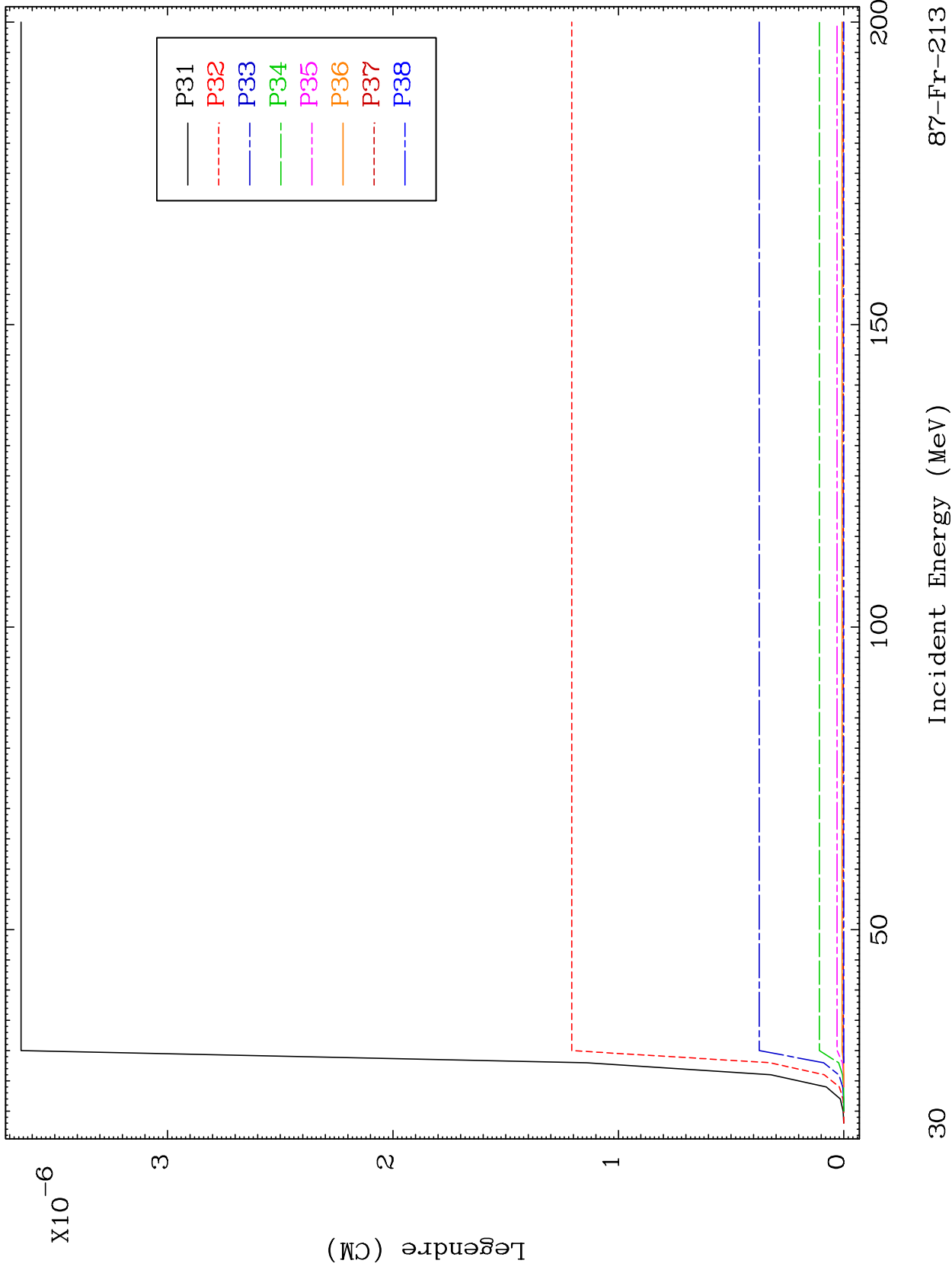
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MAT 8728

1.189 MeV (n,n') Level  
Legendre Coefficients

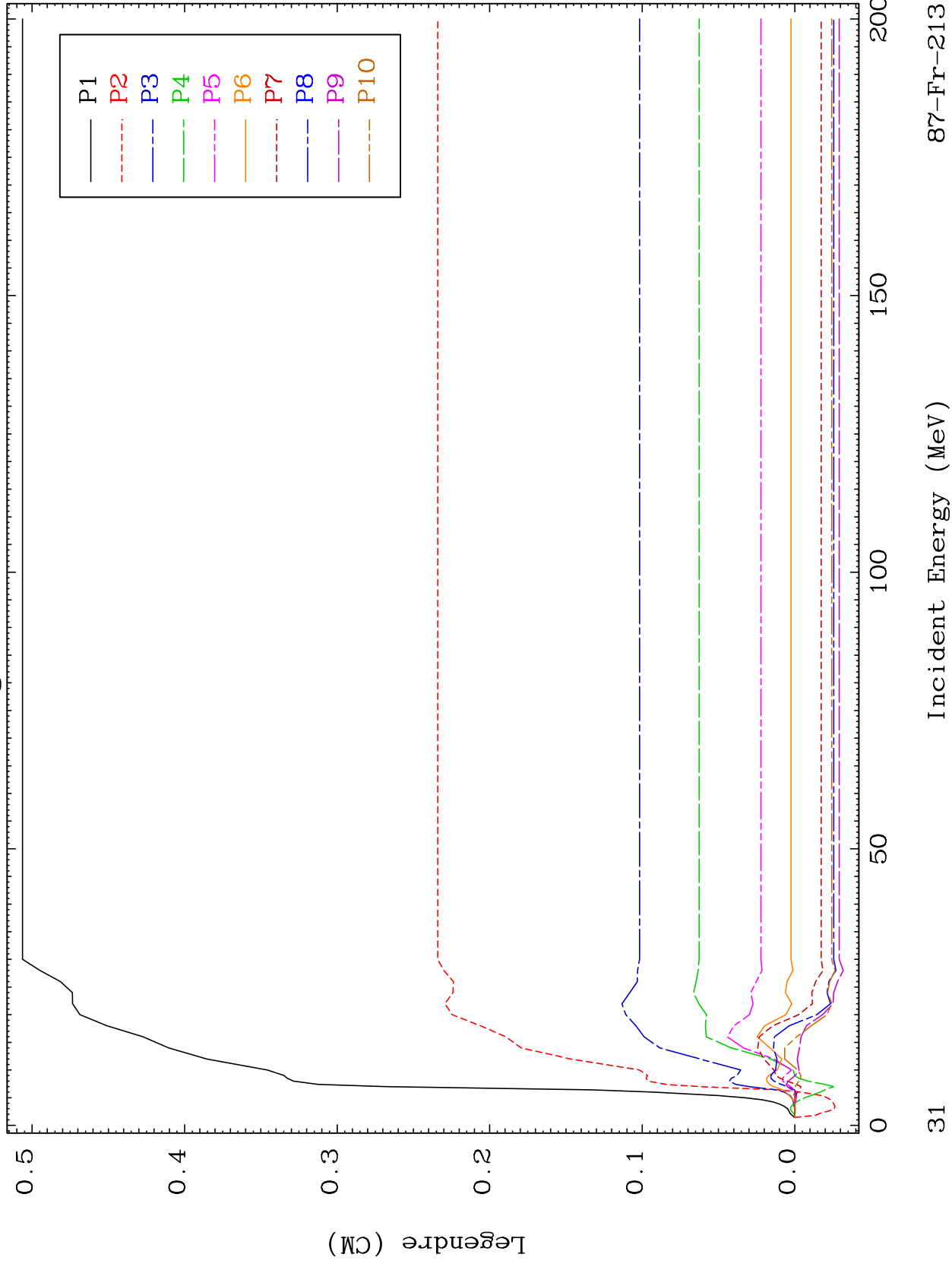
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MAT 8728

1.411 MeV (n,n') Level  
Legendre Coefficients

87-Fr-213



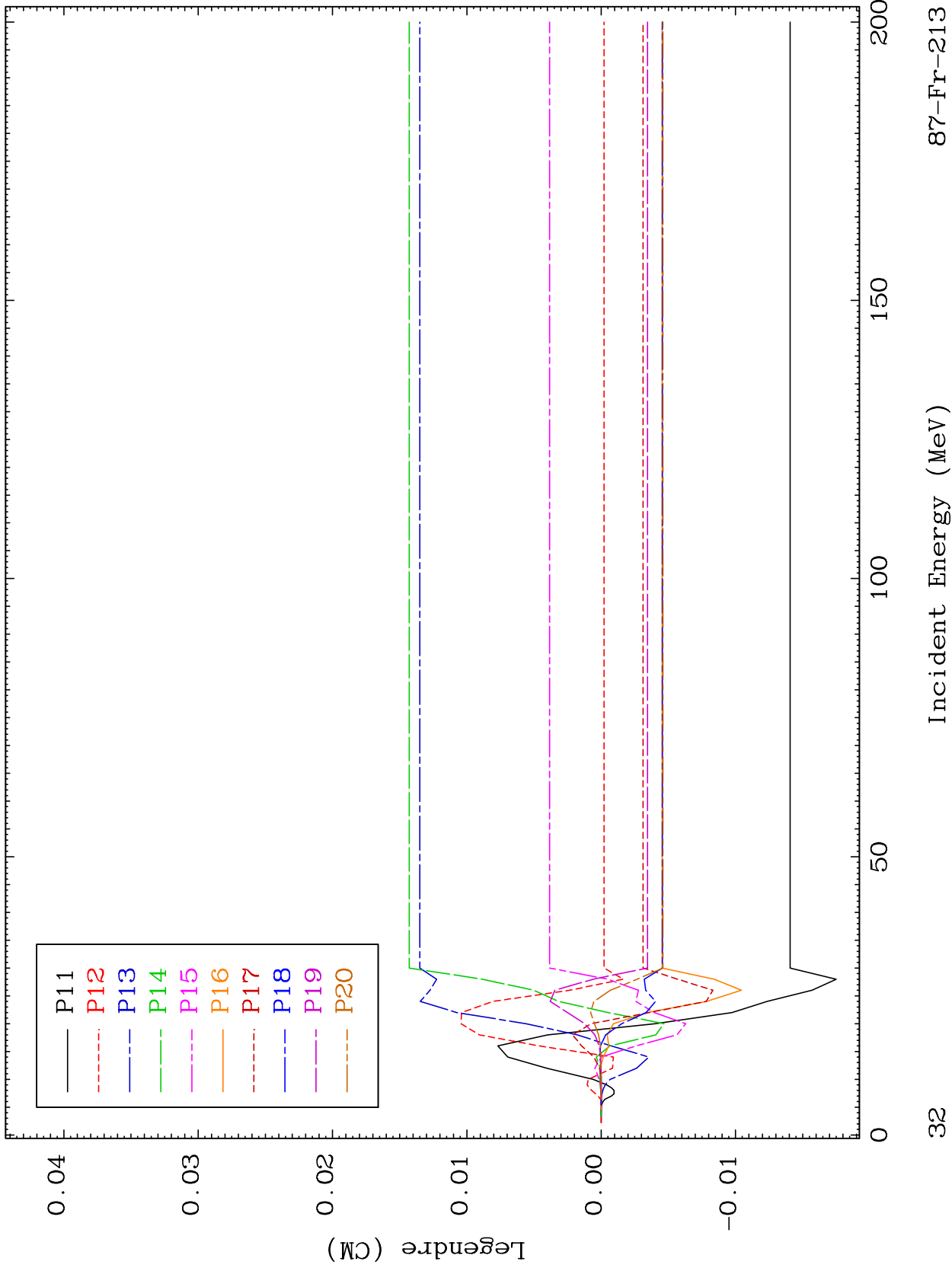
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MAT 8728

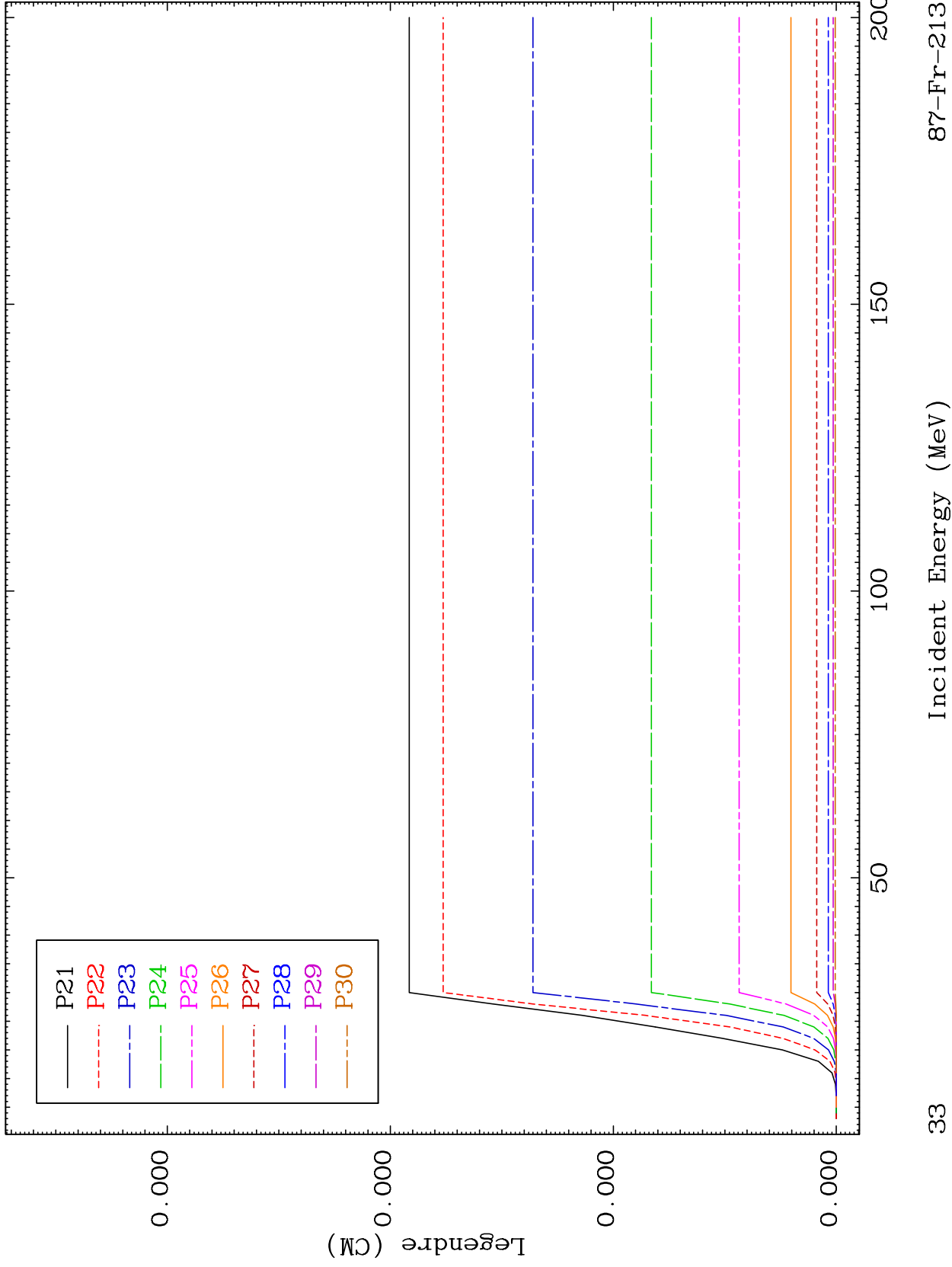
1.411 MeV (n, n') Level  
Legendre Coefficients

87-Fr-213



32

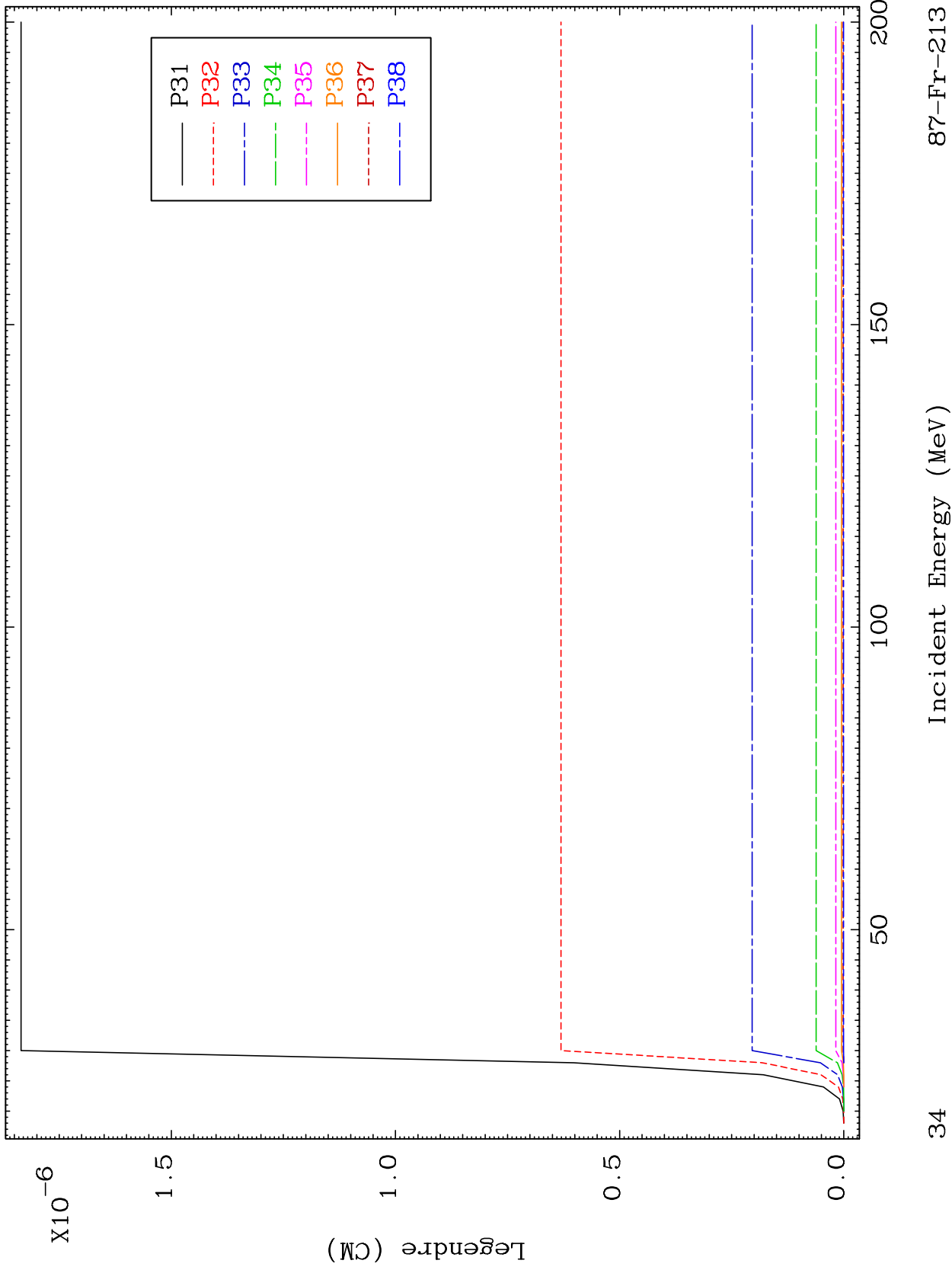
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MAT 8728

1.411 MeV (n,n') Level  
Legendre Coefficients

87-Fr-213

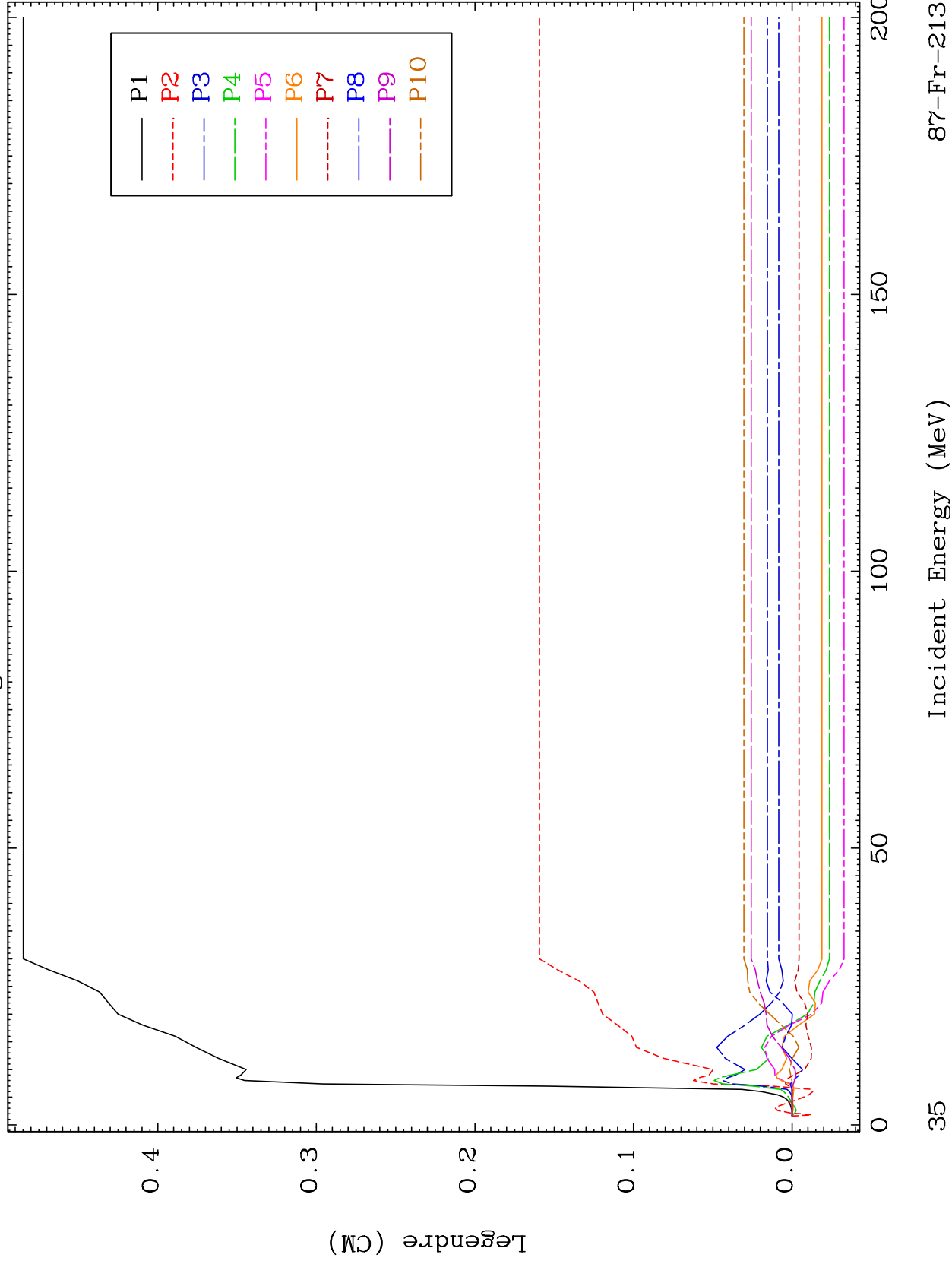


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MAT 8728

1.590 MeV (n,n') Level  
Legendre Coefficients

87-Fr-213



87-Fr-213

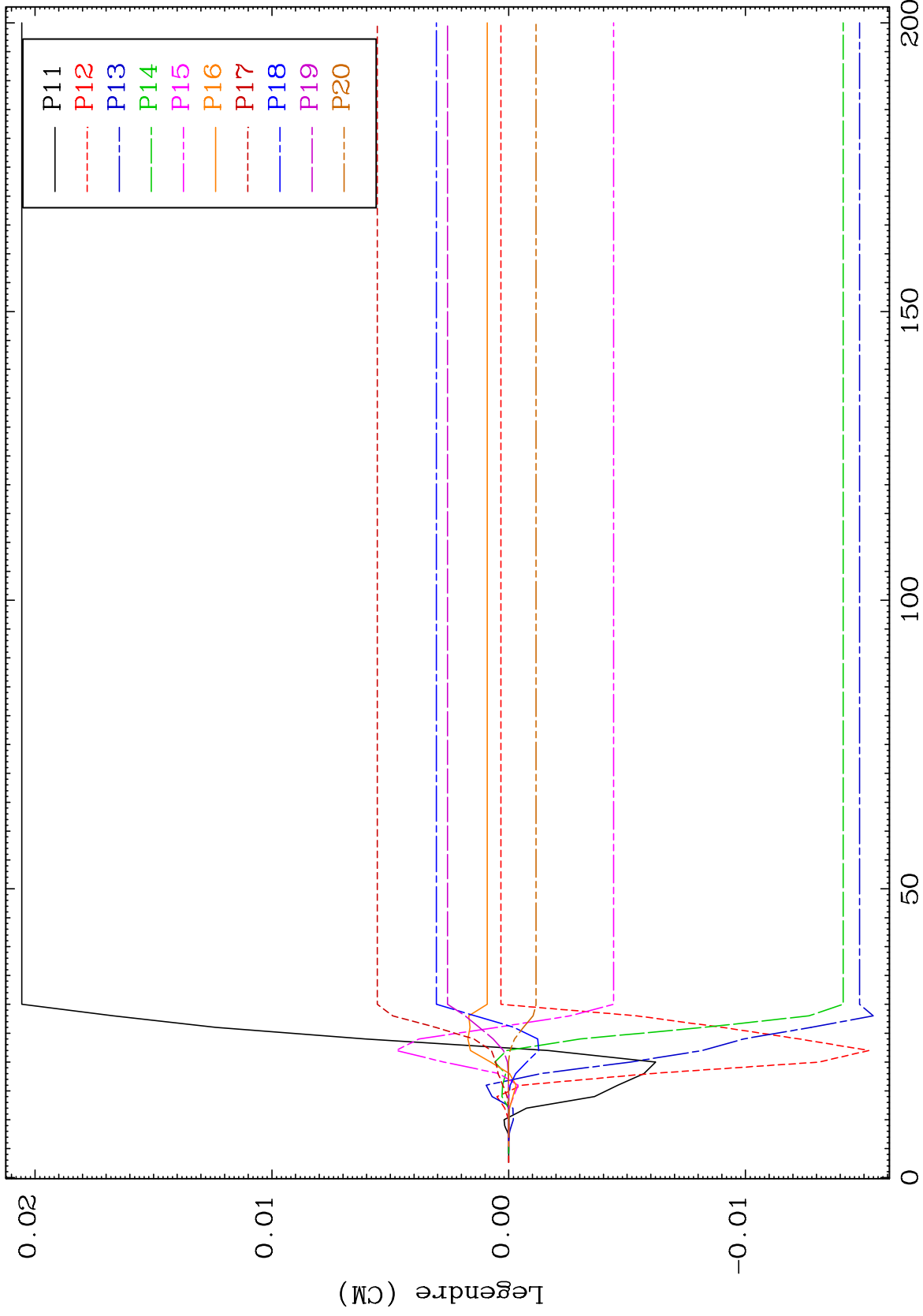
Incident Energy (MeV)

35

MAT 8728

1.590 MeV (n,n') Level  
Legendre Coefficients

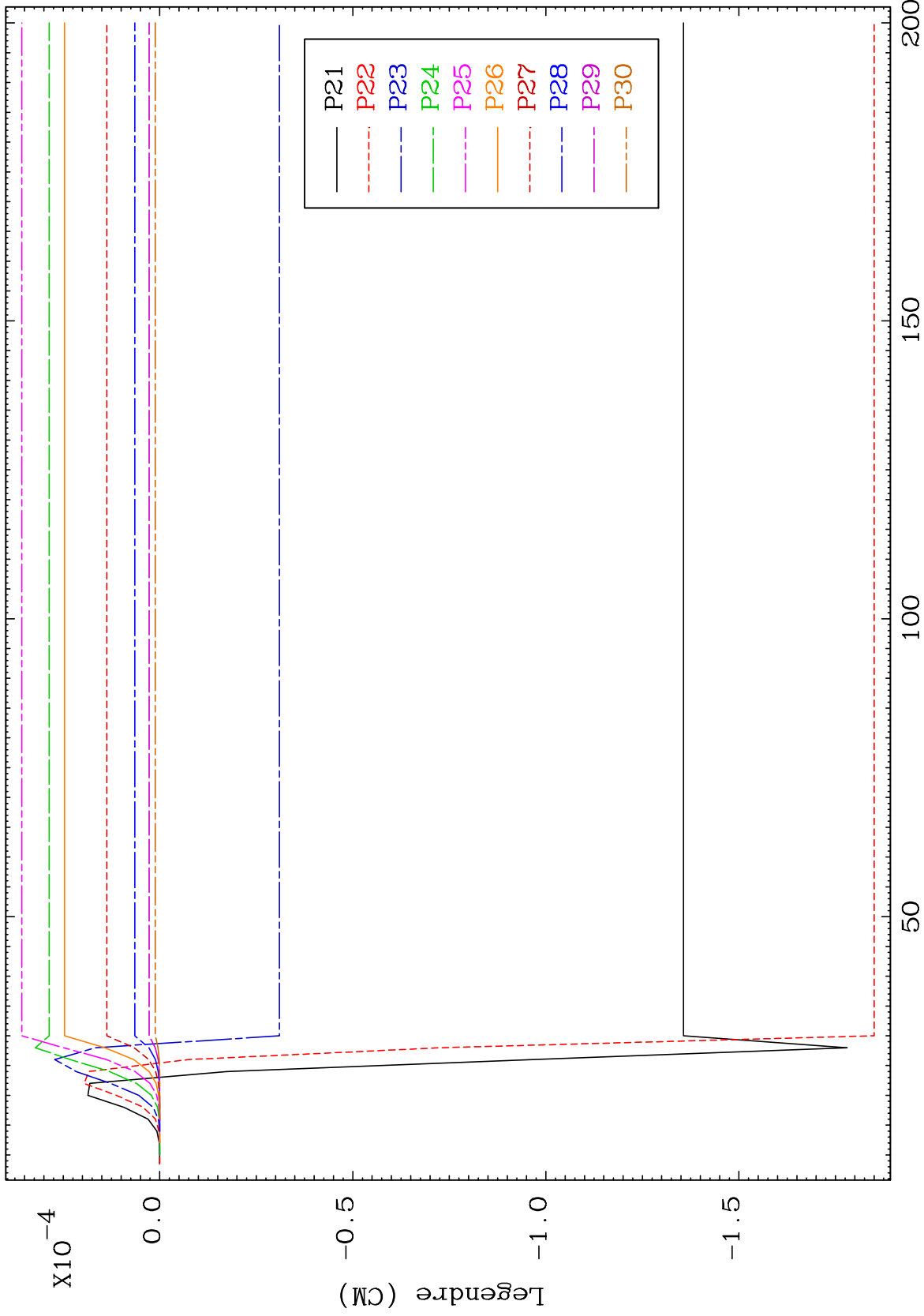
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87-Fr-213

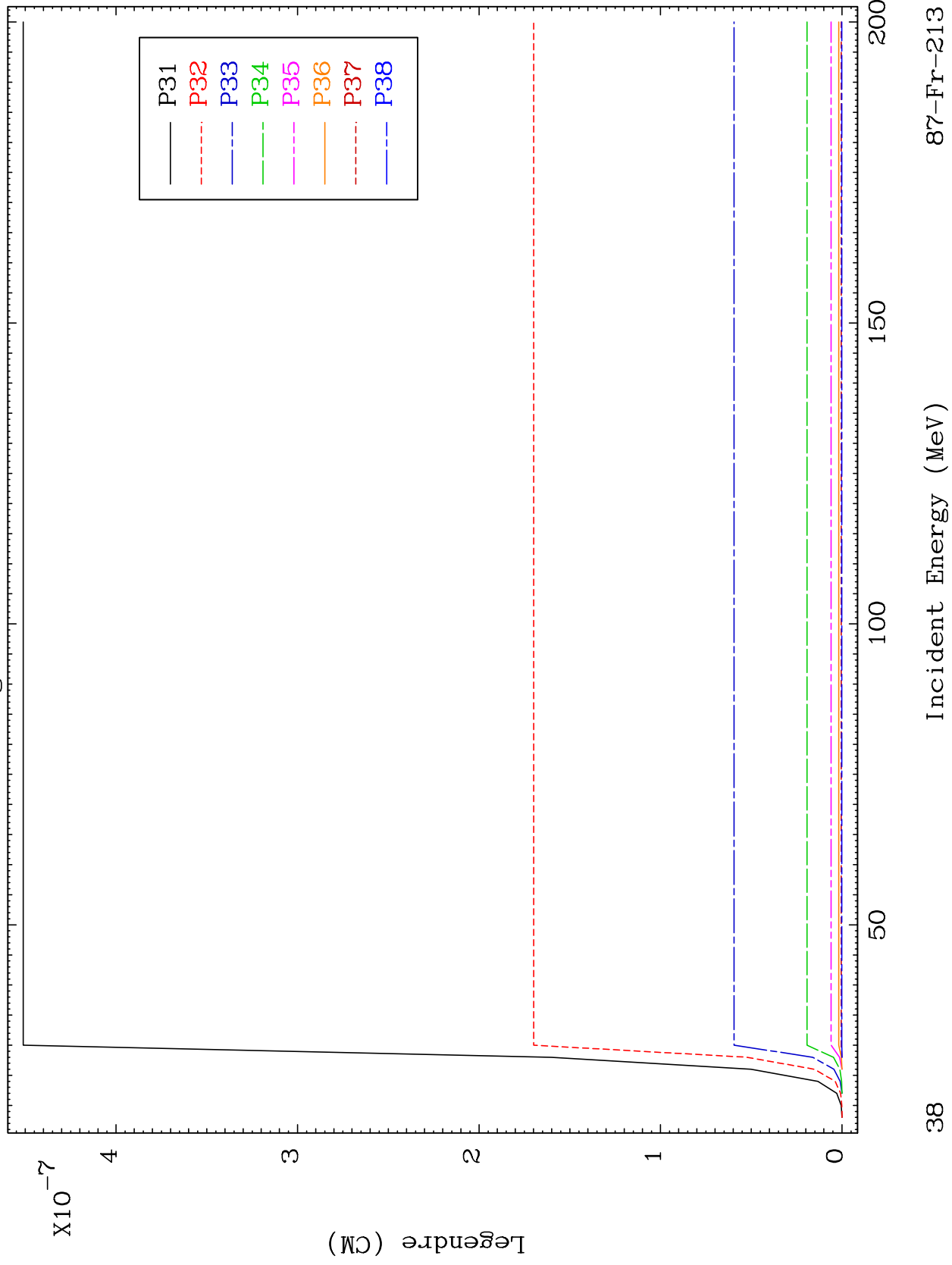
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MAT 8728

1.590 MeV (n,n') Level  
Legendre Coefficients

87-Fr-213

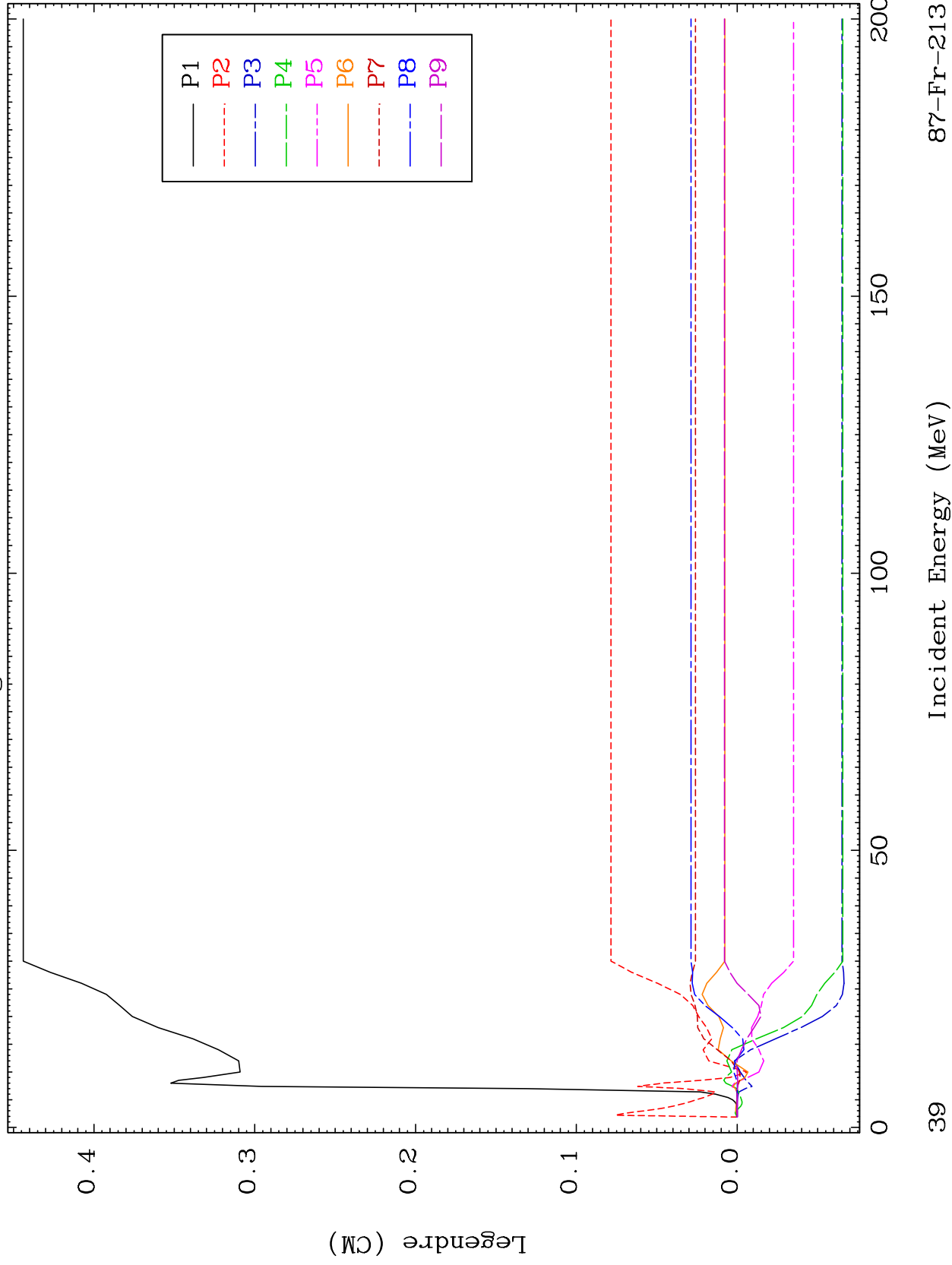


38

MAT 8728

1.856 MeV (n,n') Level  
Legendre Coefficients

87-Fr-213



87-Fr-213

Incident Energy (MeV)

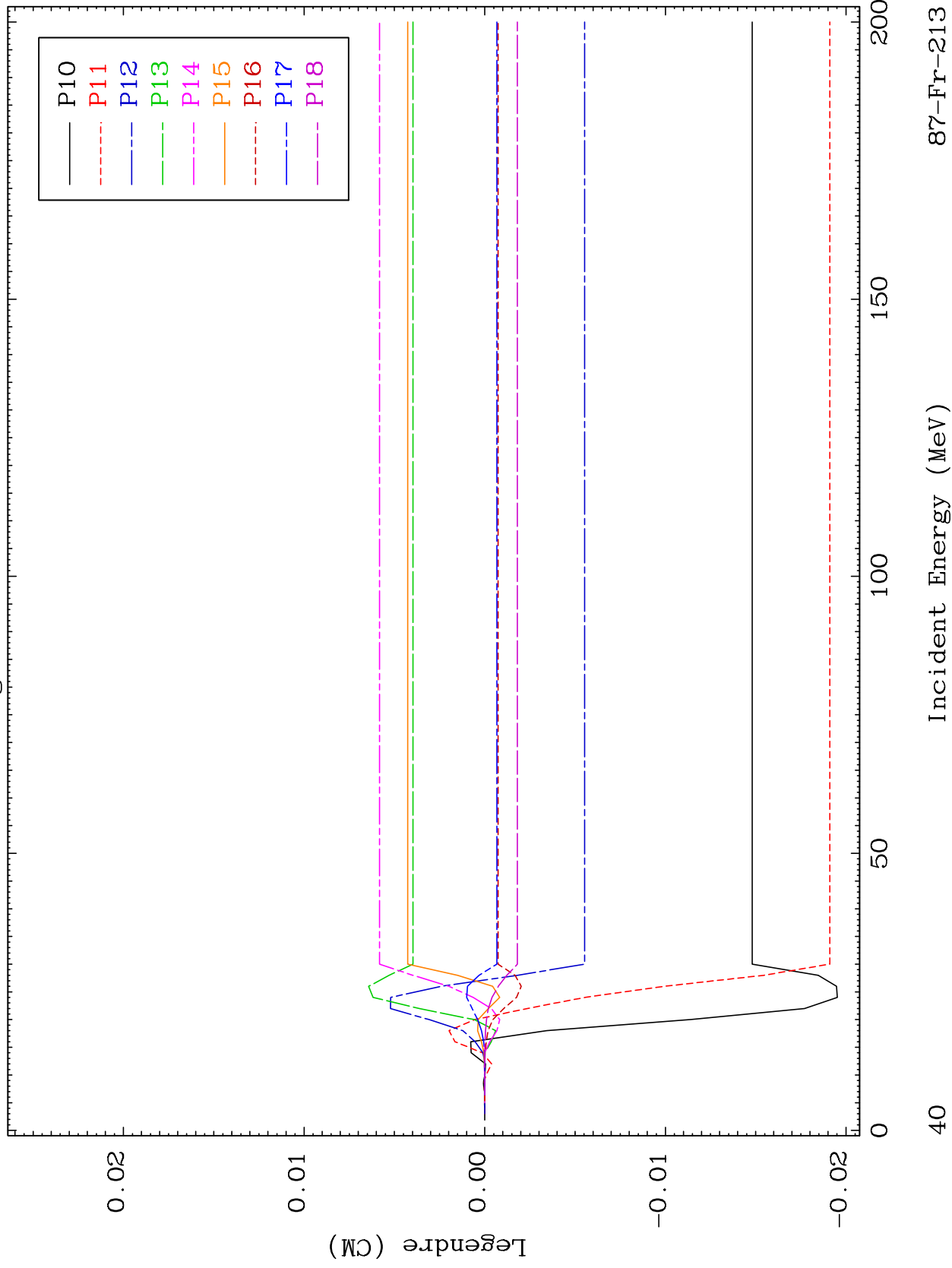
39



MAT 8728

1.856 MeV (n,n') Level  
Legendre Coefficients

87-Fr-213

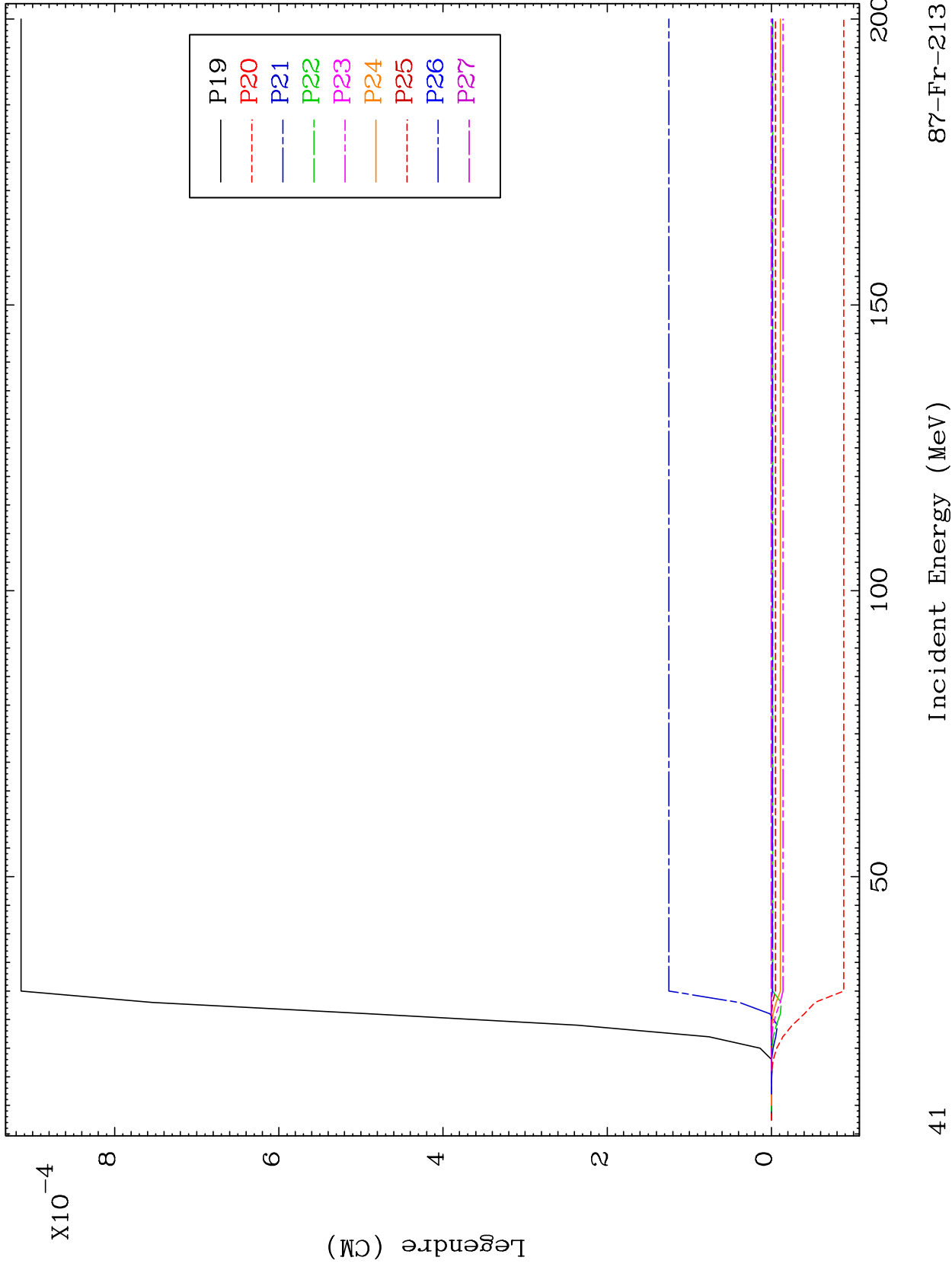


87-Fr-213

MAT 8728

1.856 MeV (n,n') Level  
Legendre Coefficients

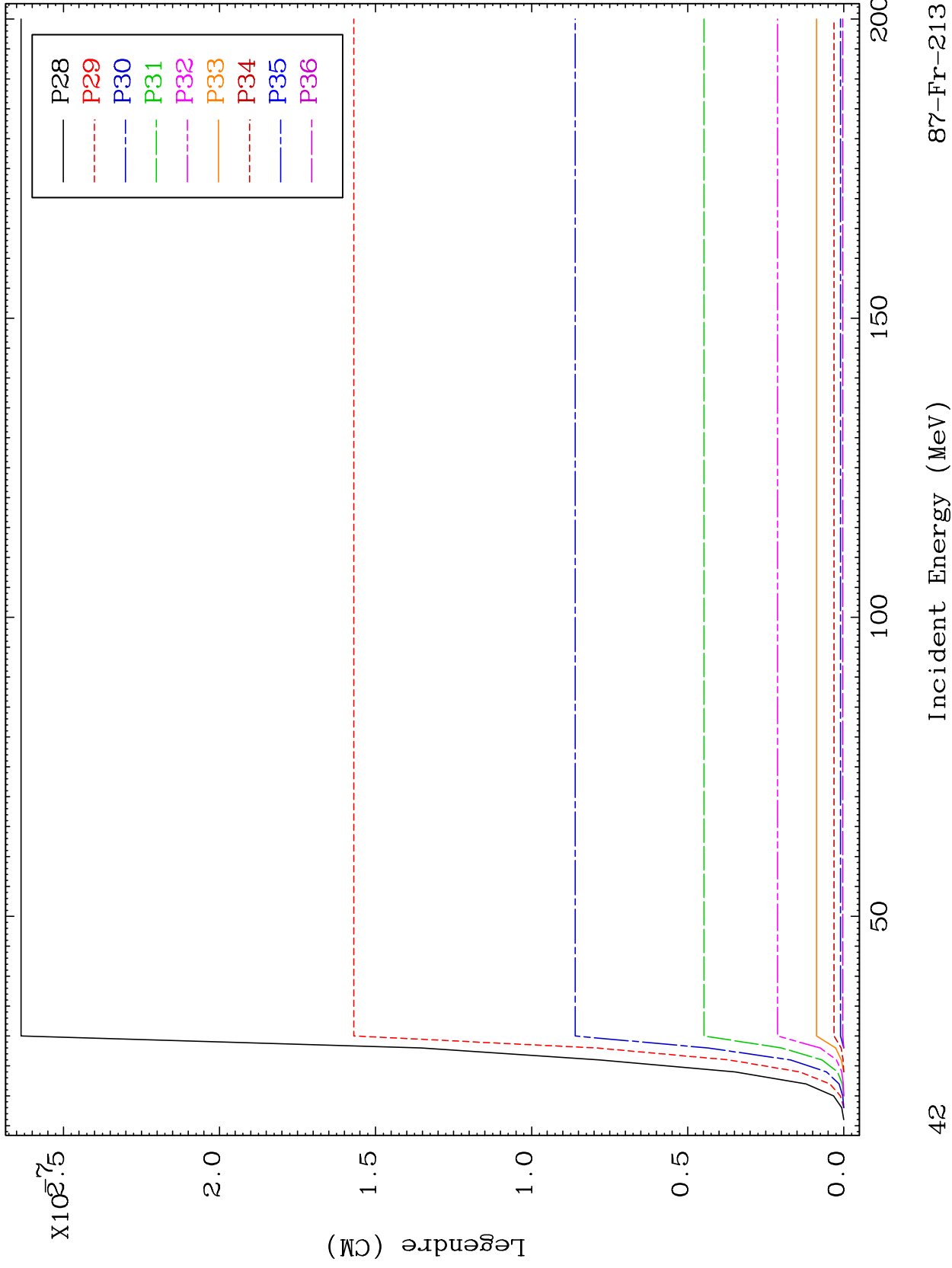
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MAT 8728

1.856 MeV (n,n') Level  
Legendre Coefficients

87-Fr-213



42

Incident Energy (MeV)

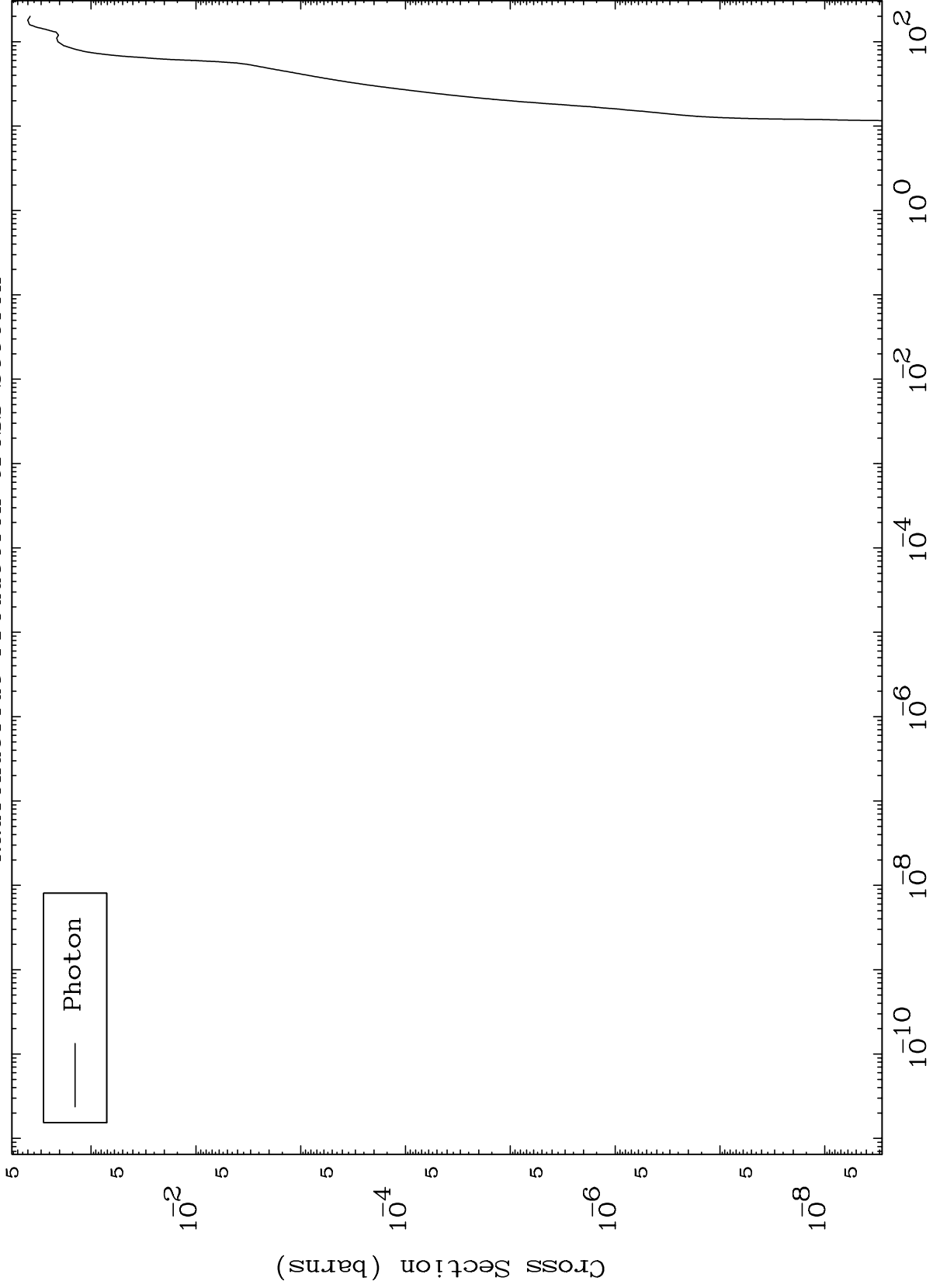
87-Fr-213

MAT 8728

Fission

87-Fr-213

Radionuclide Production Cross Section



43

Incident Energy (MeV)

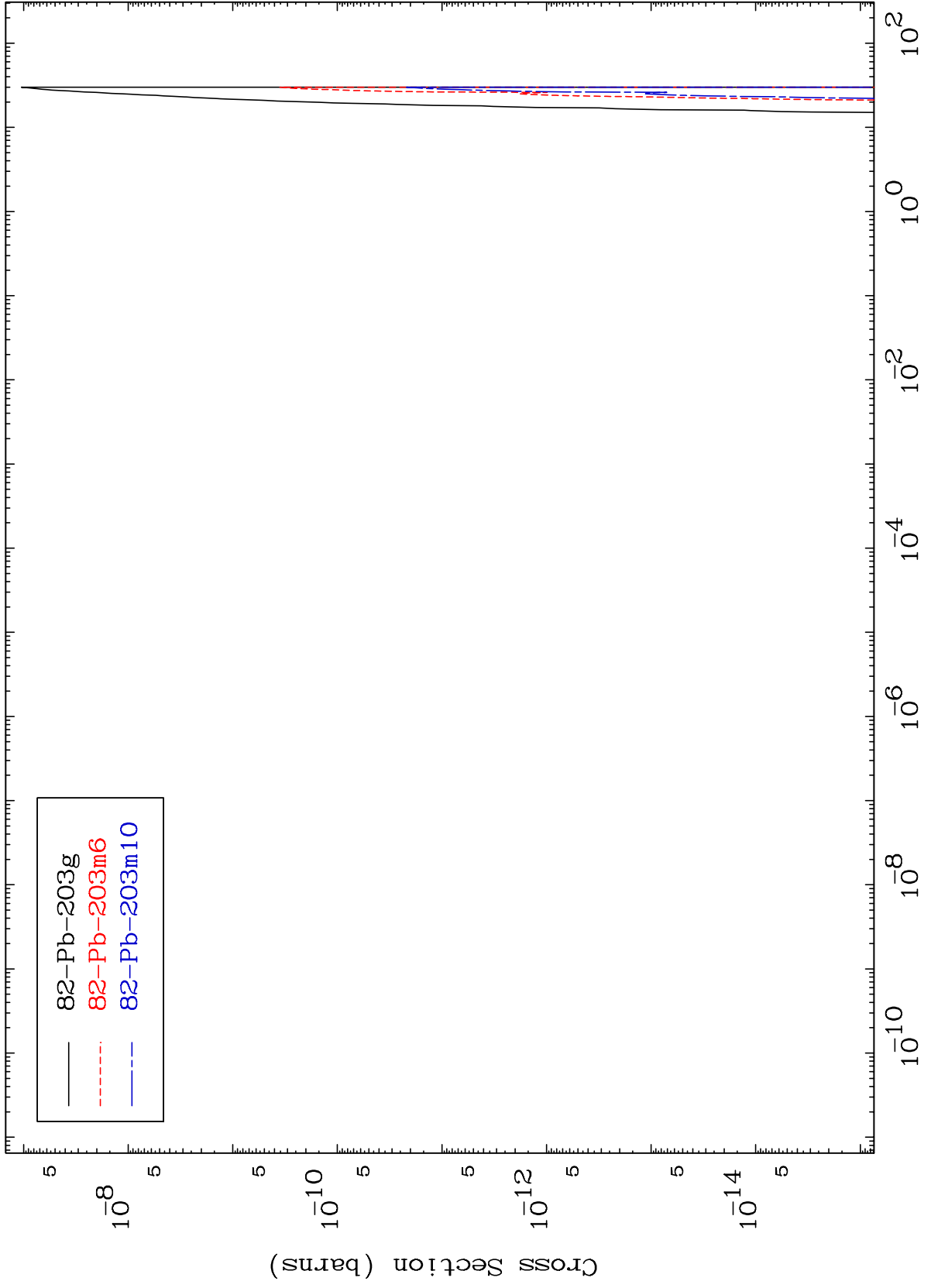
87-Fr-213

MAT 8728

(n,n') d,2α

87-Fr-213

Radionuclide Production Cross Section



44

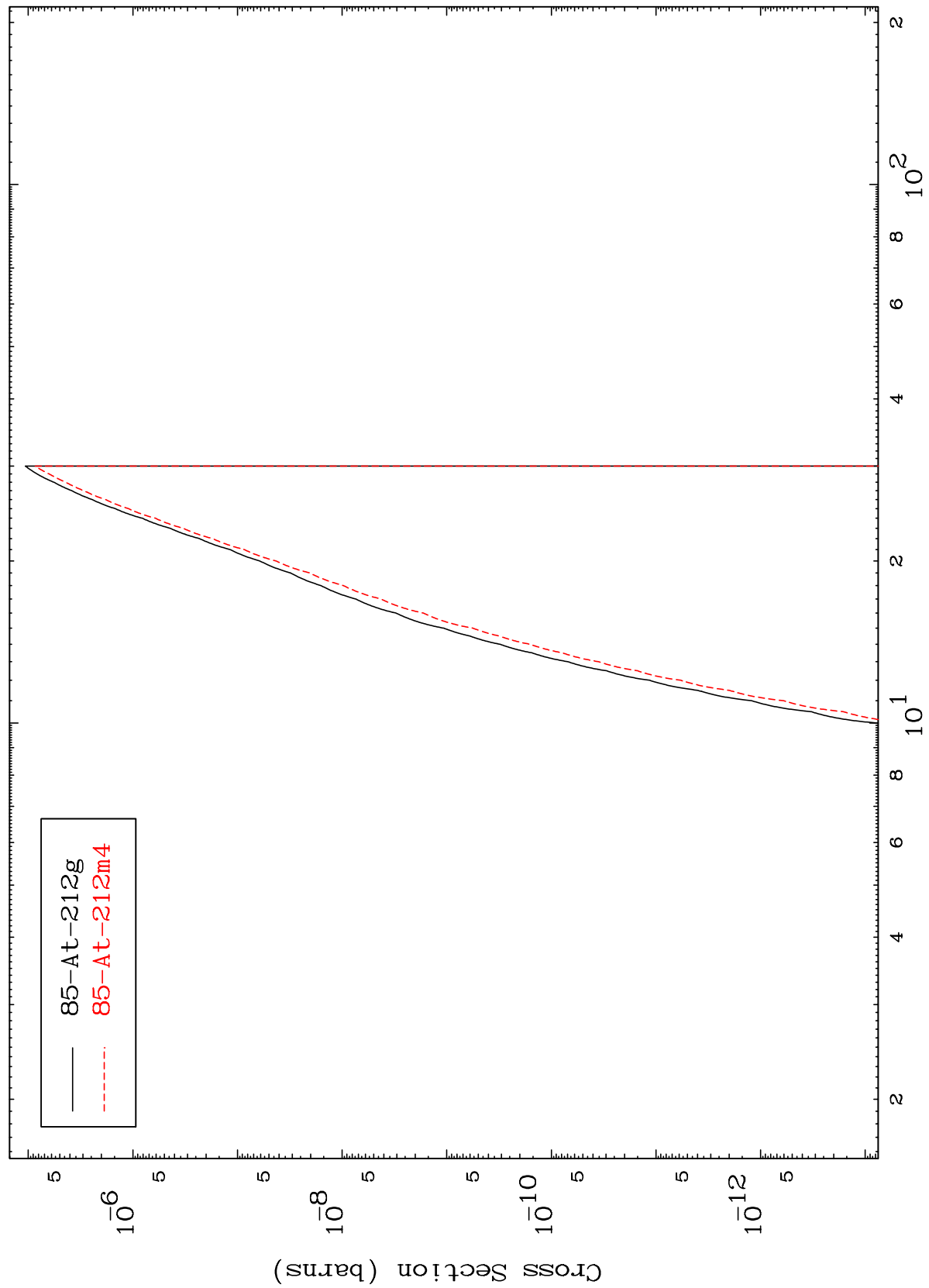
Incident Energy (MeV)

87-Fr-213

MAT 8728

87-Fr-213

(n,2p)  
Radionuclide Production Cross Section



— 85-At-212g  
- - - 85-At-212m4

45

Incident Energy (MeV)

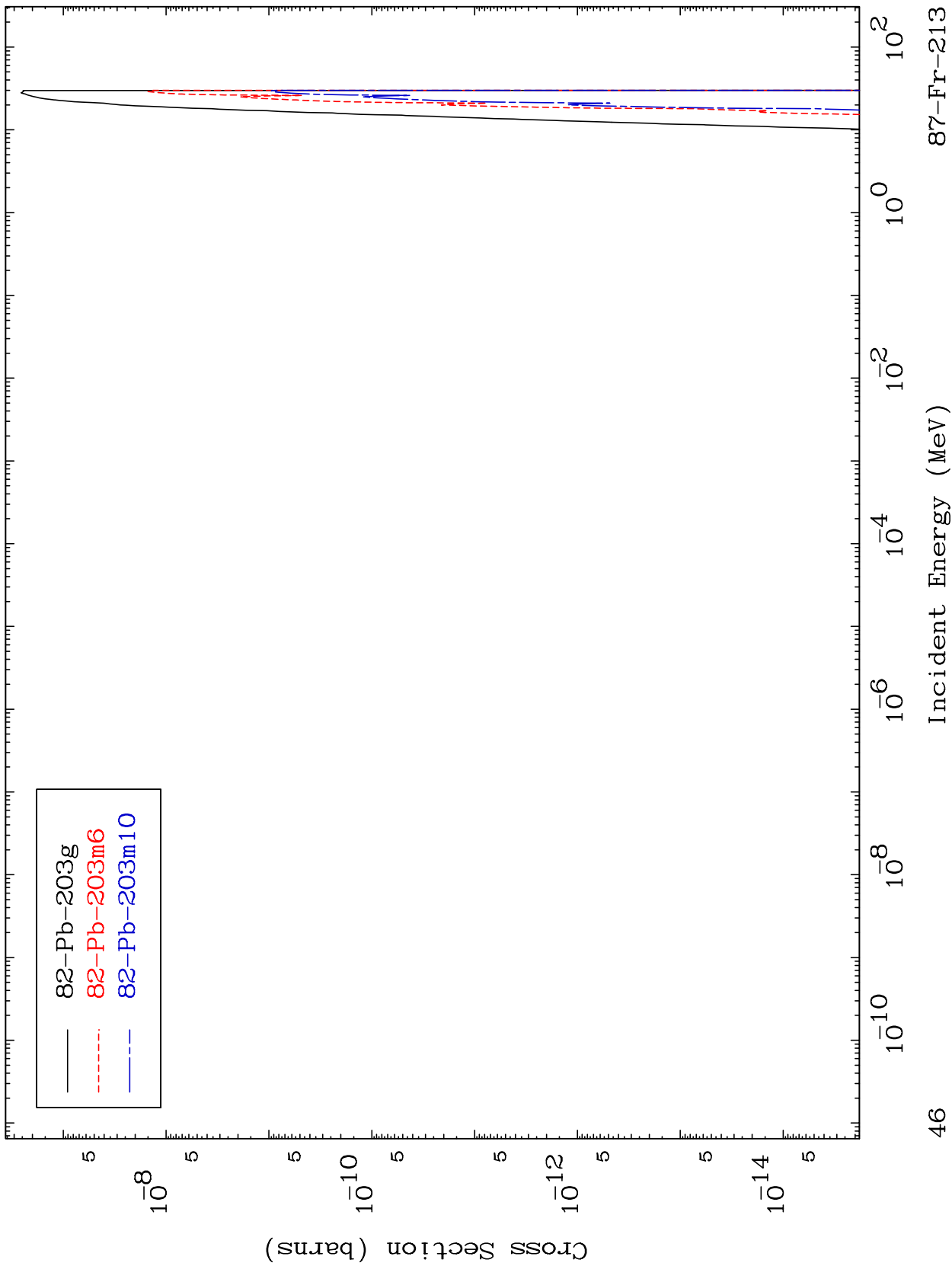
87-Fr-213

MAT 8728

(n,t) 2 $\alpha$

87-Fr-213

Radionuclide Production Cross Section



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Incident Energy (MeV)

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