

Program EVALPLOT
(Version 2015-2)

by

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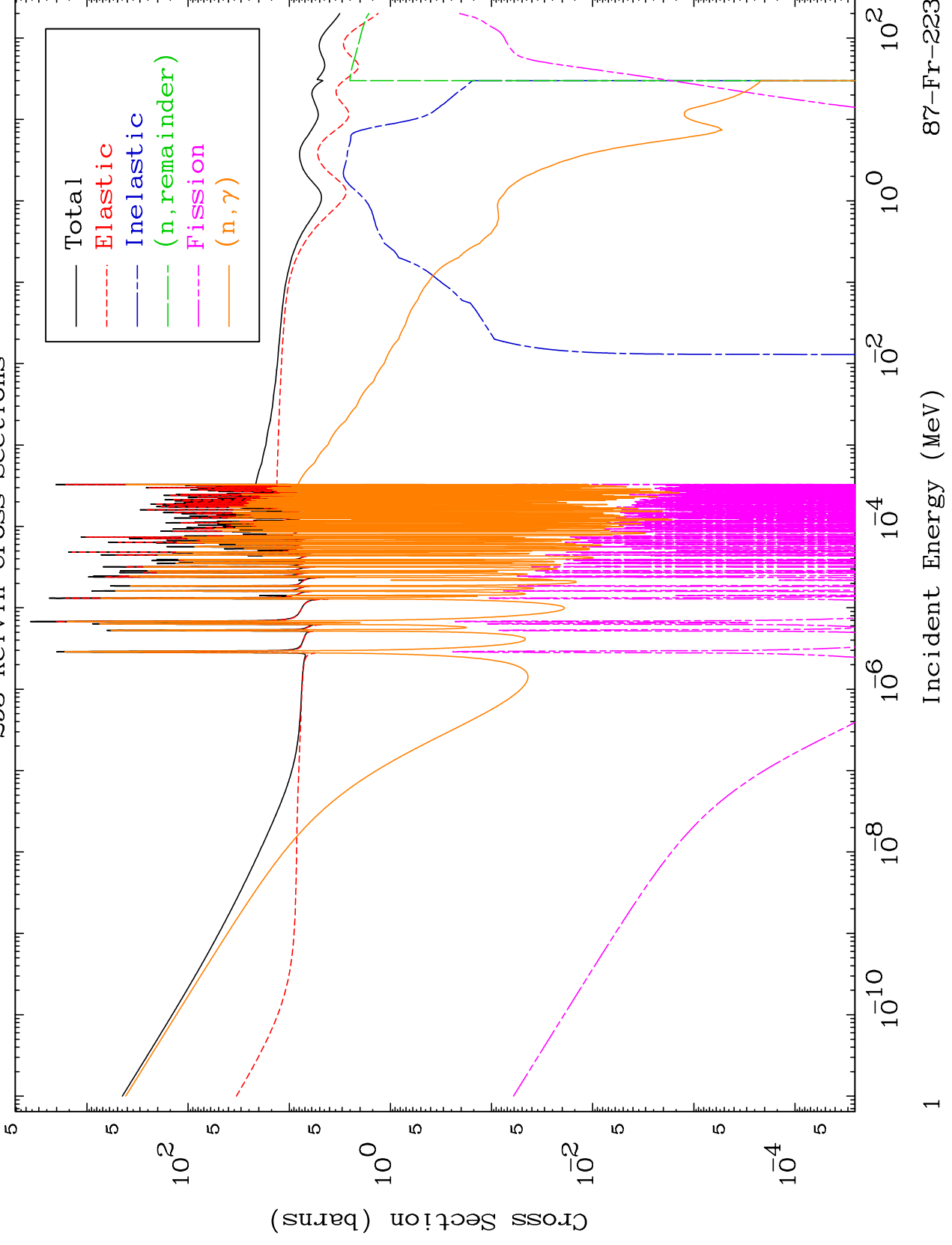
Press Mouse Button to Start

MAT 8758

Major

293 Kelvin Cross Sections

87-Fr-223



87-Fr-223

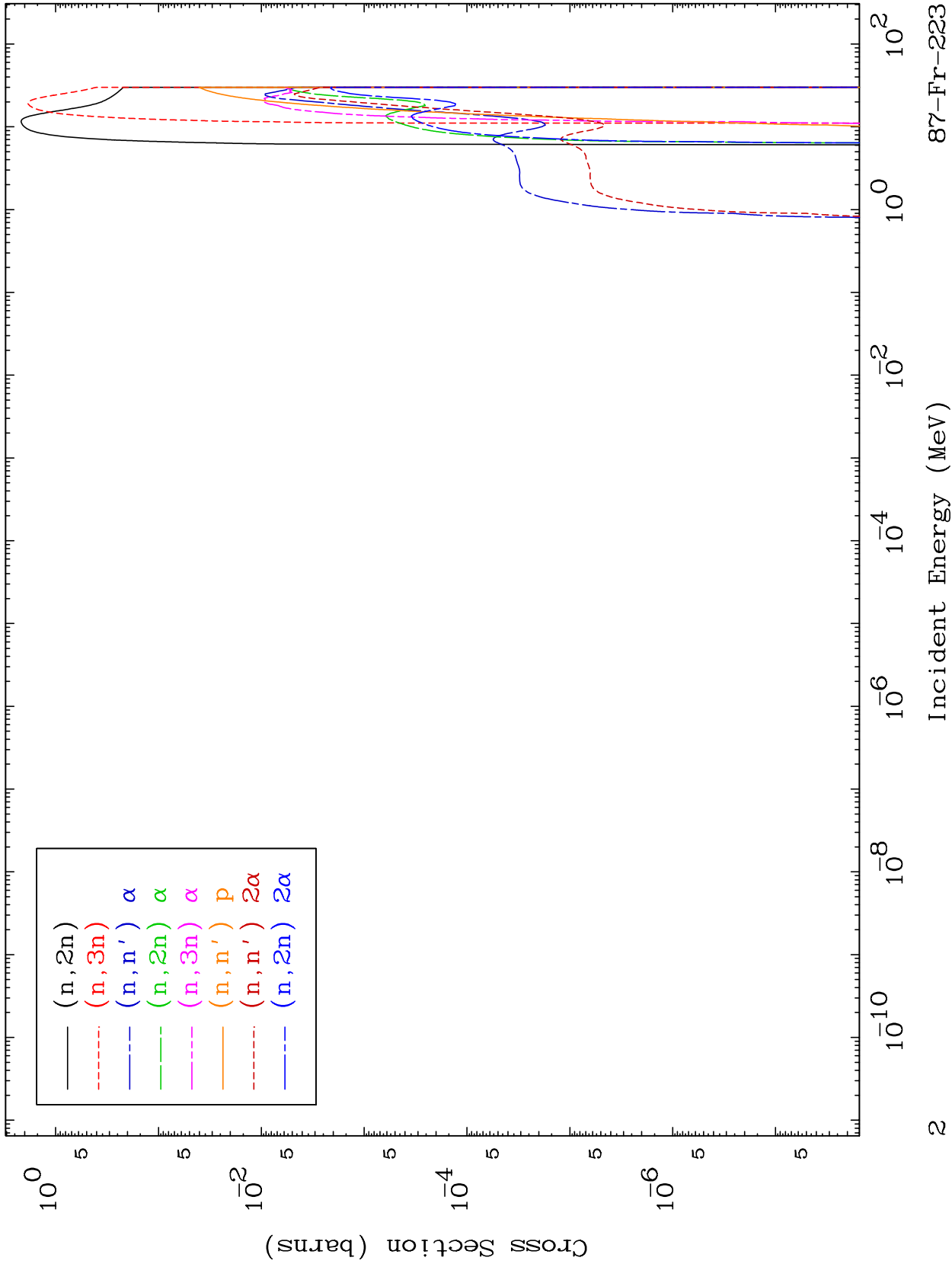
Incident Energy (MeV)

1

MAT 8758

Neutron Production
293 Kelvin Cross Sections

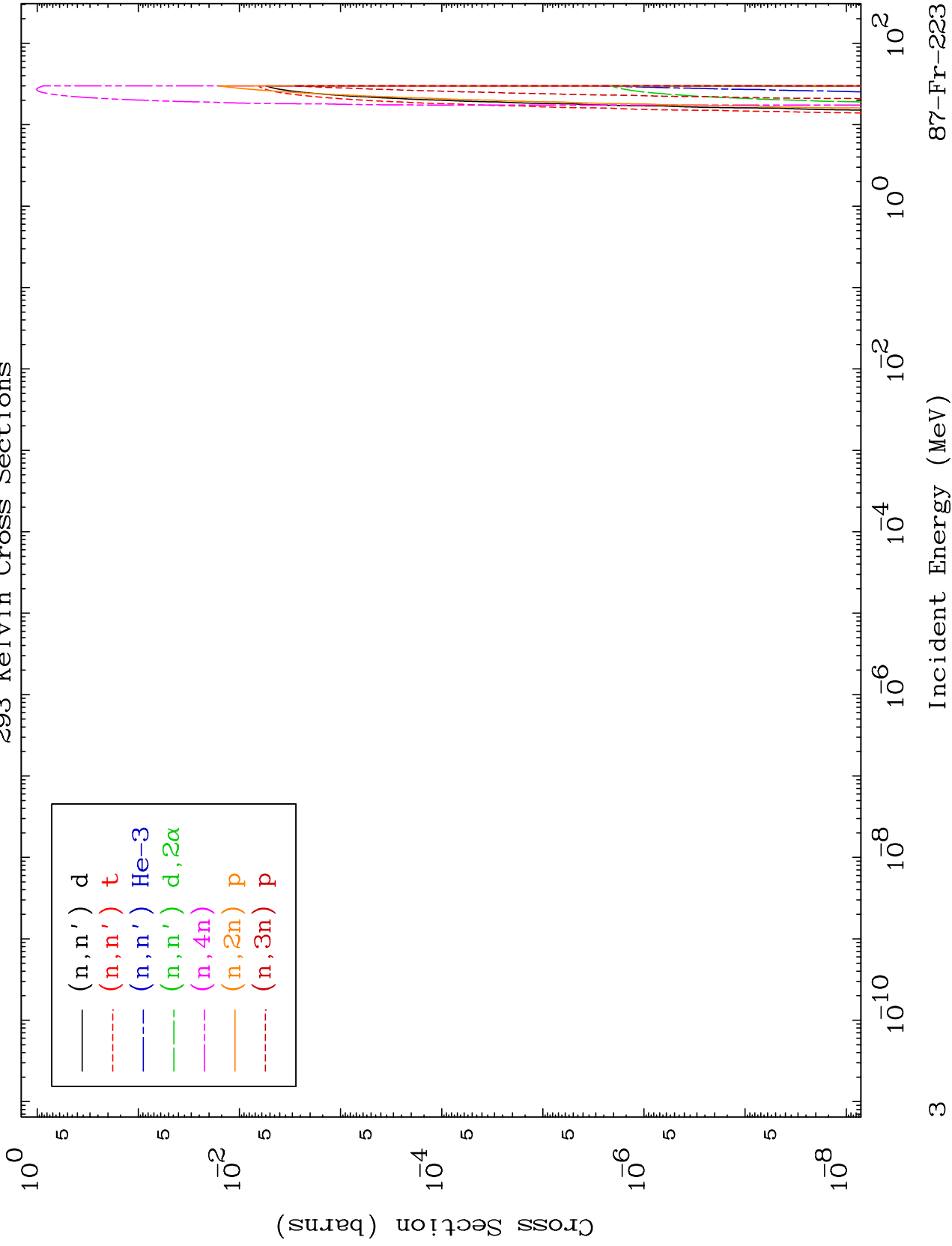
87-Fr-223



MAT 8758

Neutron Production
293 Kelvin Cross Sections

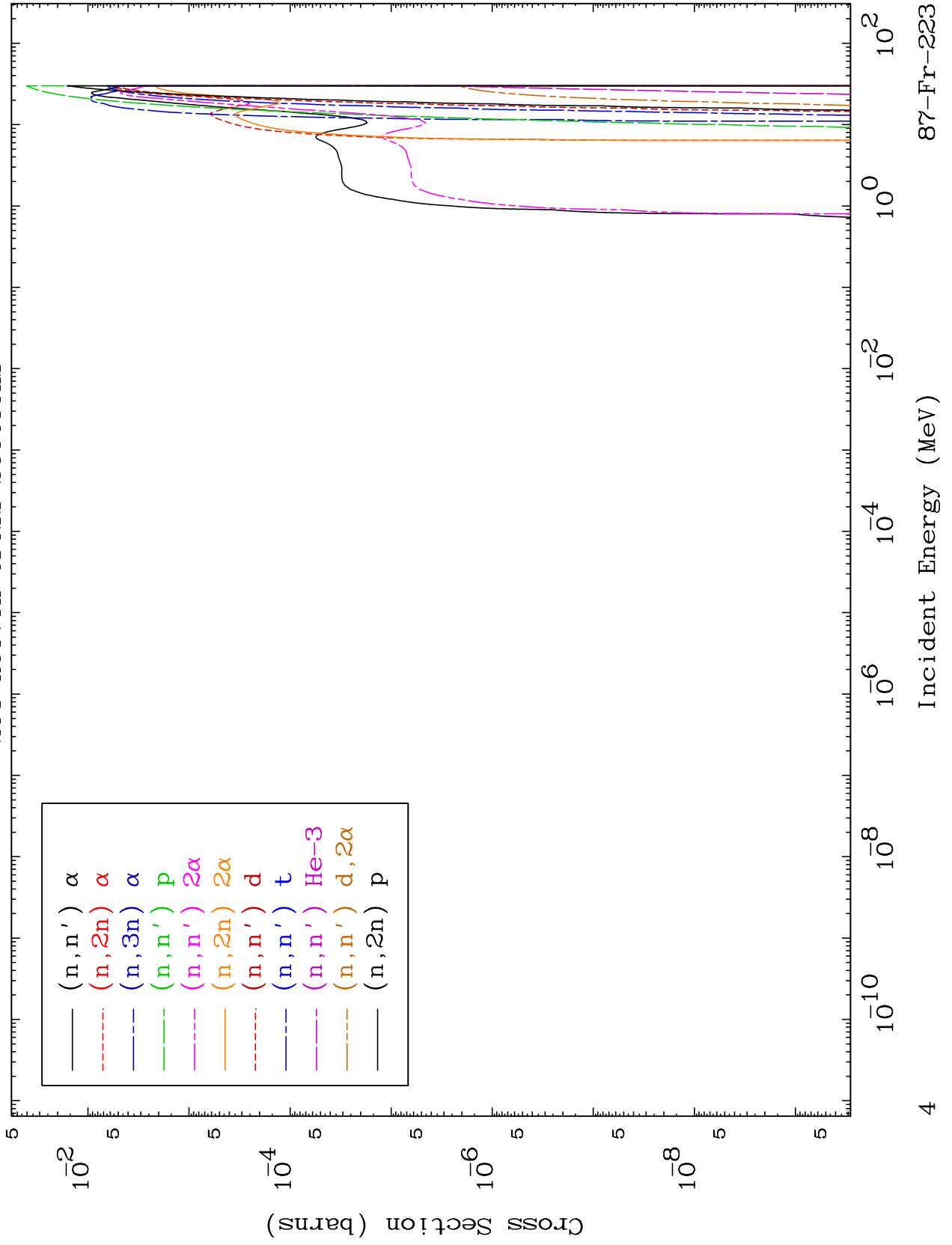
87-Fr-223



MAT 8758

Charged Particle
293 Kelvin Cross Sections

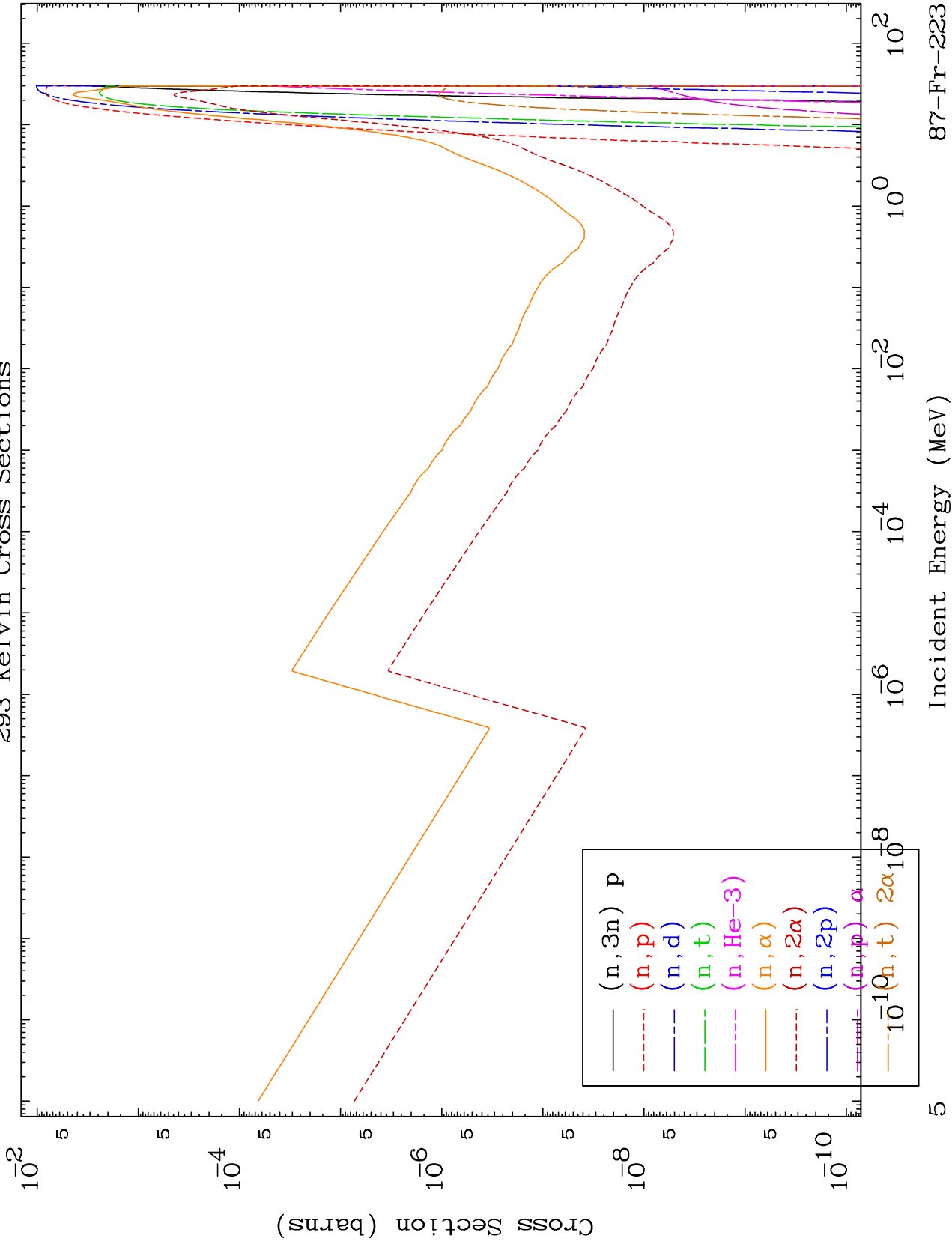
87-Fr-223



MAT 8758

Charged Particle
293 Kelvin Cross Sections

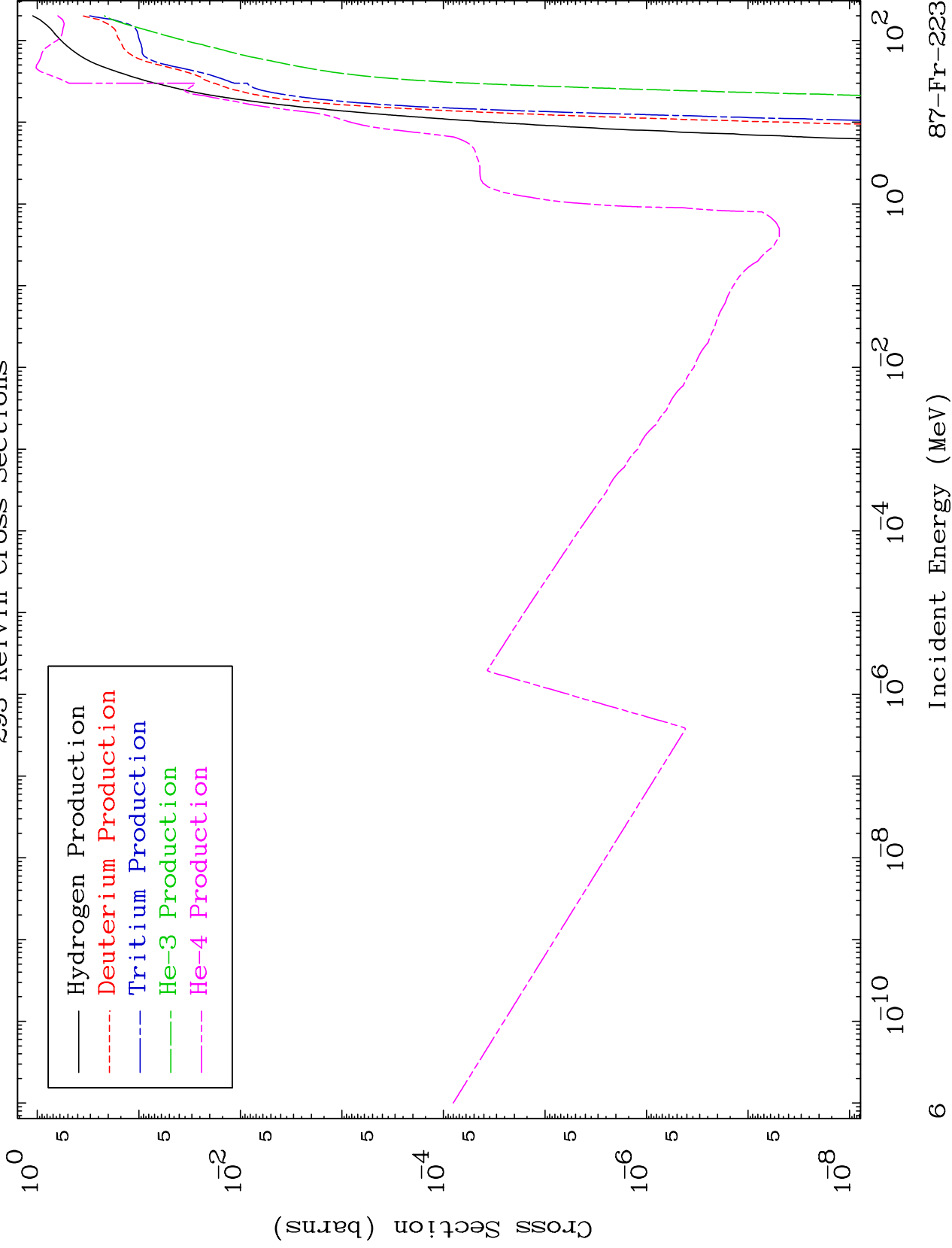
87-Fr-223



MAT 8758

Particle Production
293 Kelvin Cross Sections

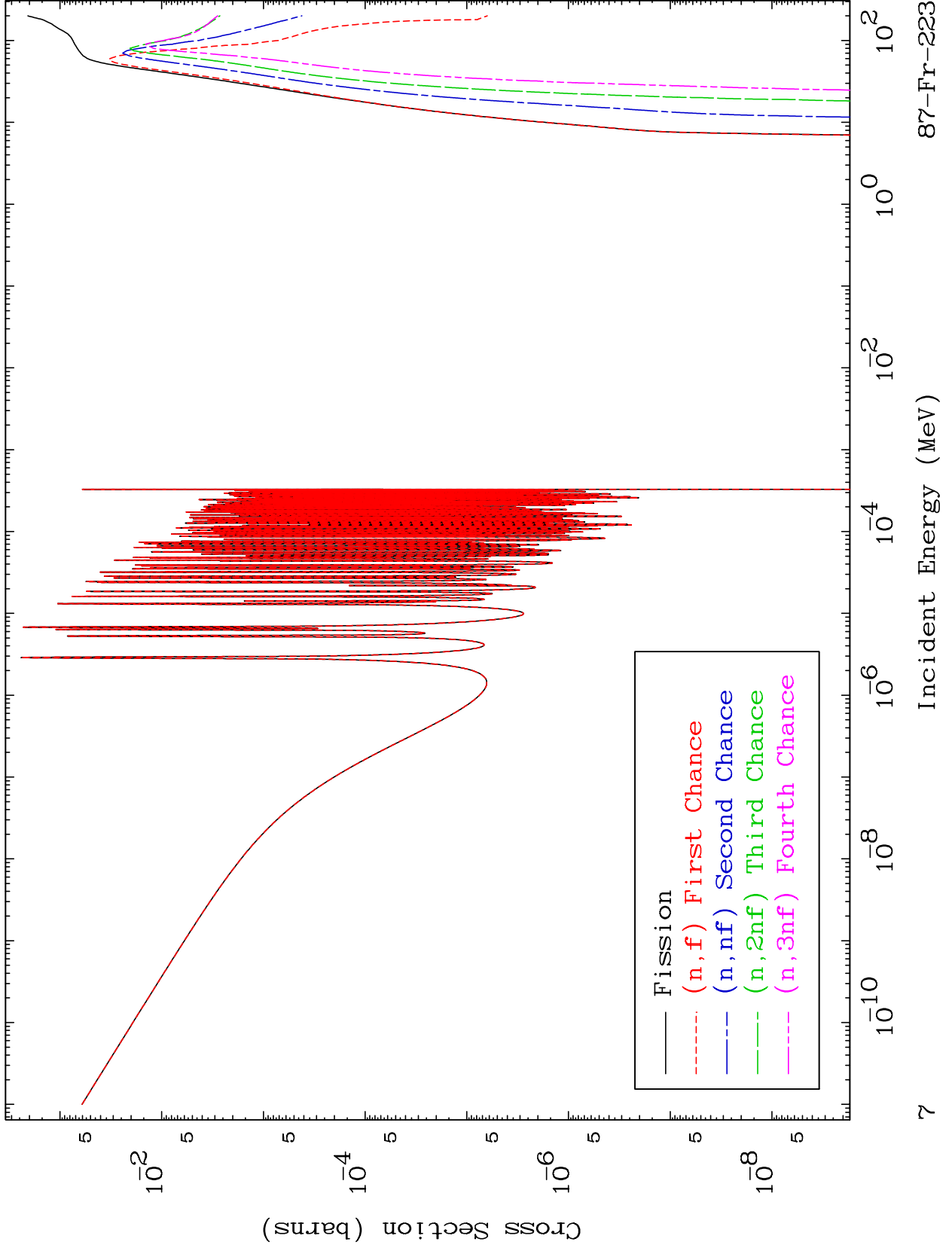
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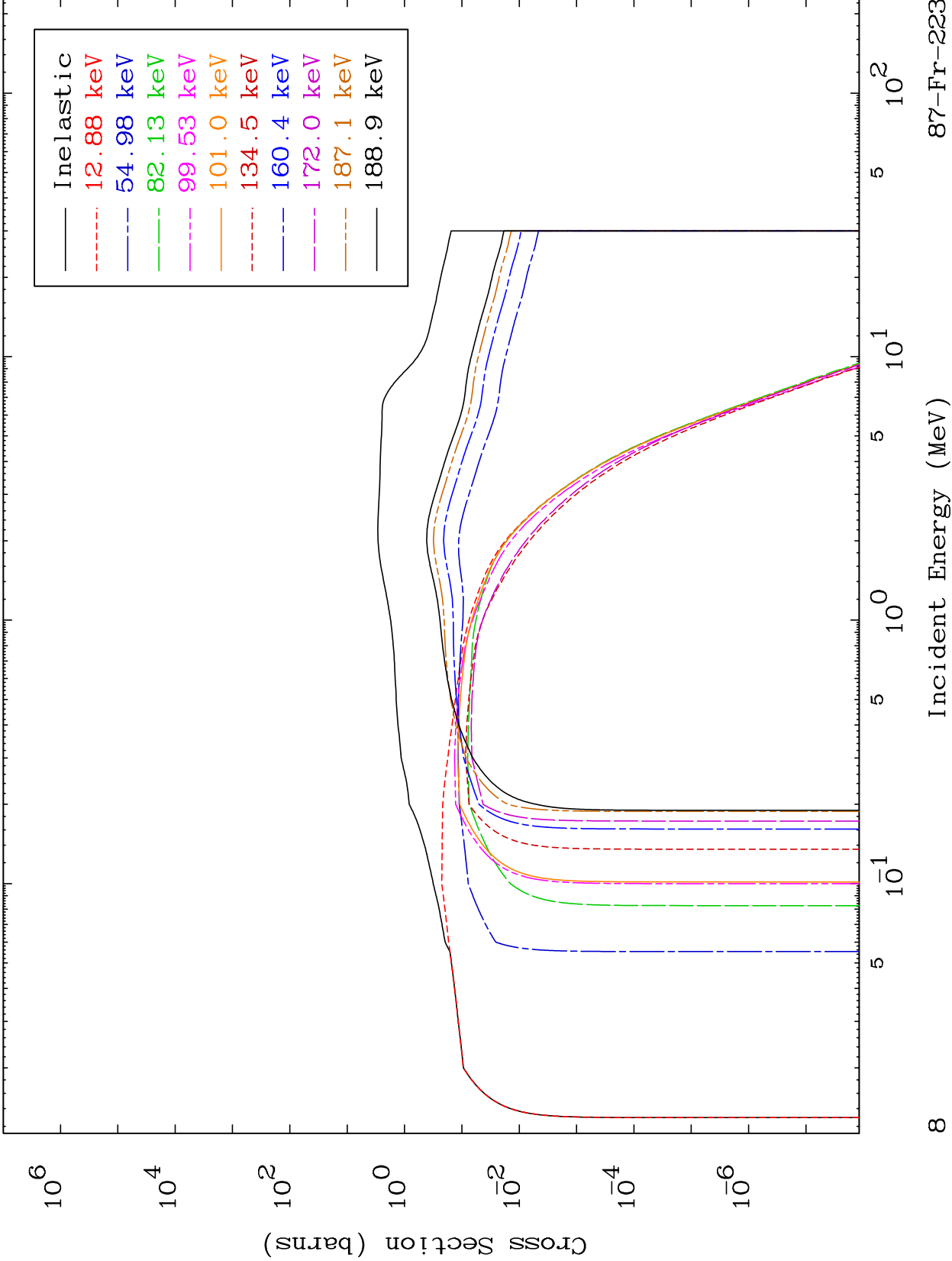


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Fission
293 Kelvin Cross Sections

87-Fr-223



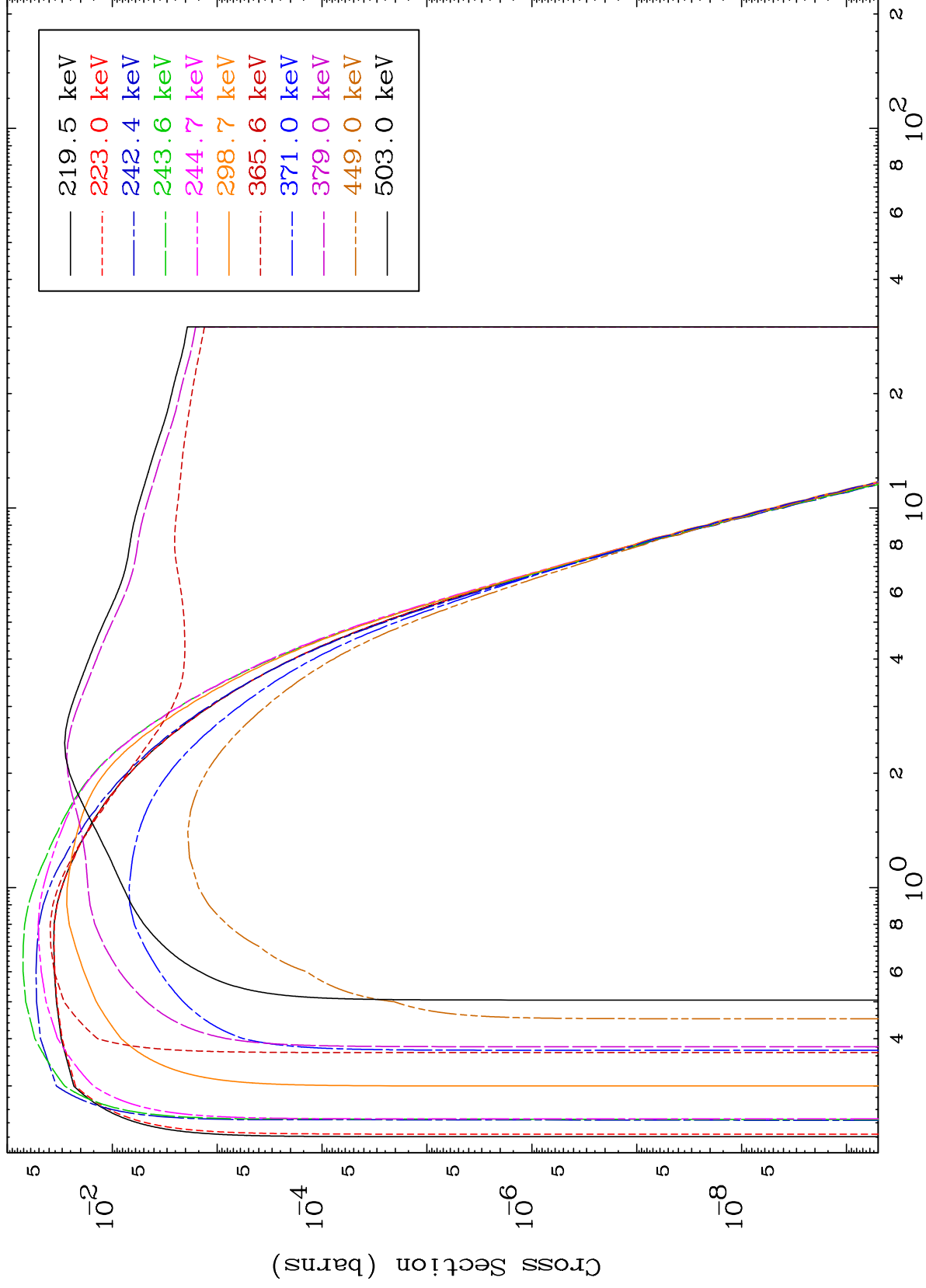


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(n,n') Level

87-Fr-223

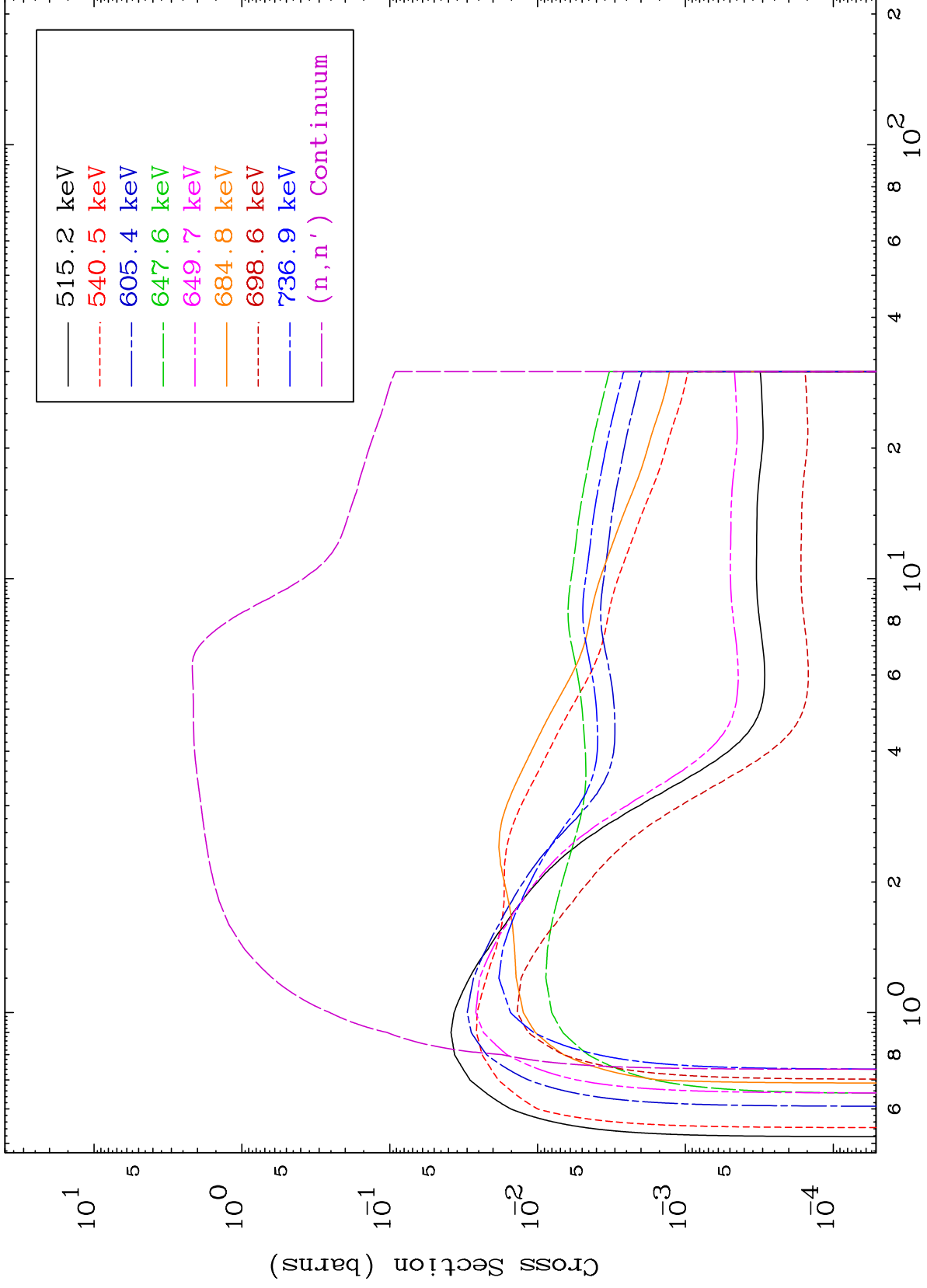
293 Kelvin Cross Sections



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(n,n') Level
293 Kelvin Cross Sections

87-Fr-223



10

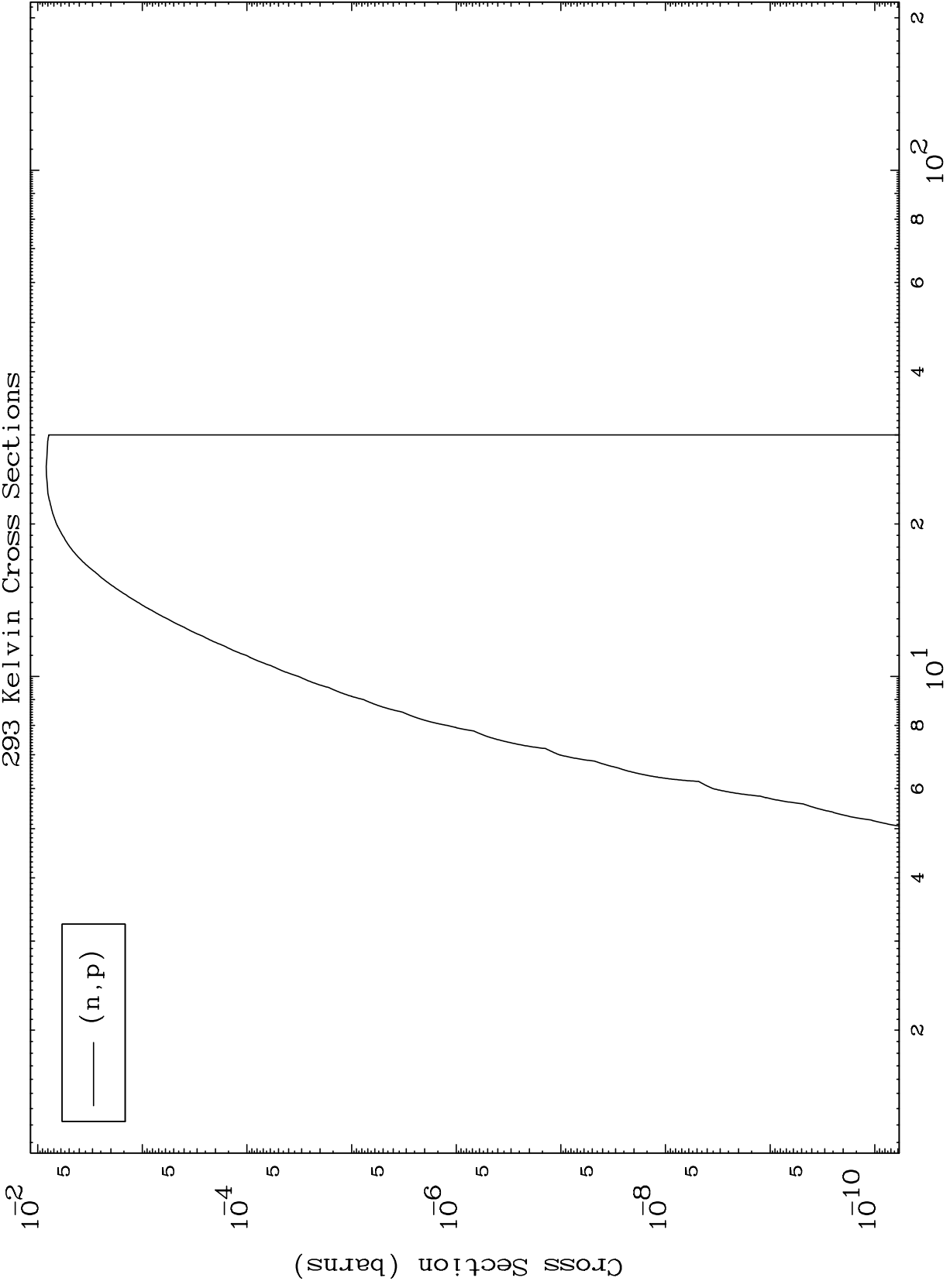
Incident Energy (MeV)

87-Fr-223

MAT 8758

(n,p) Levels
293 Kelvin Cross Sections

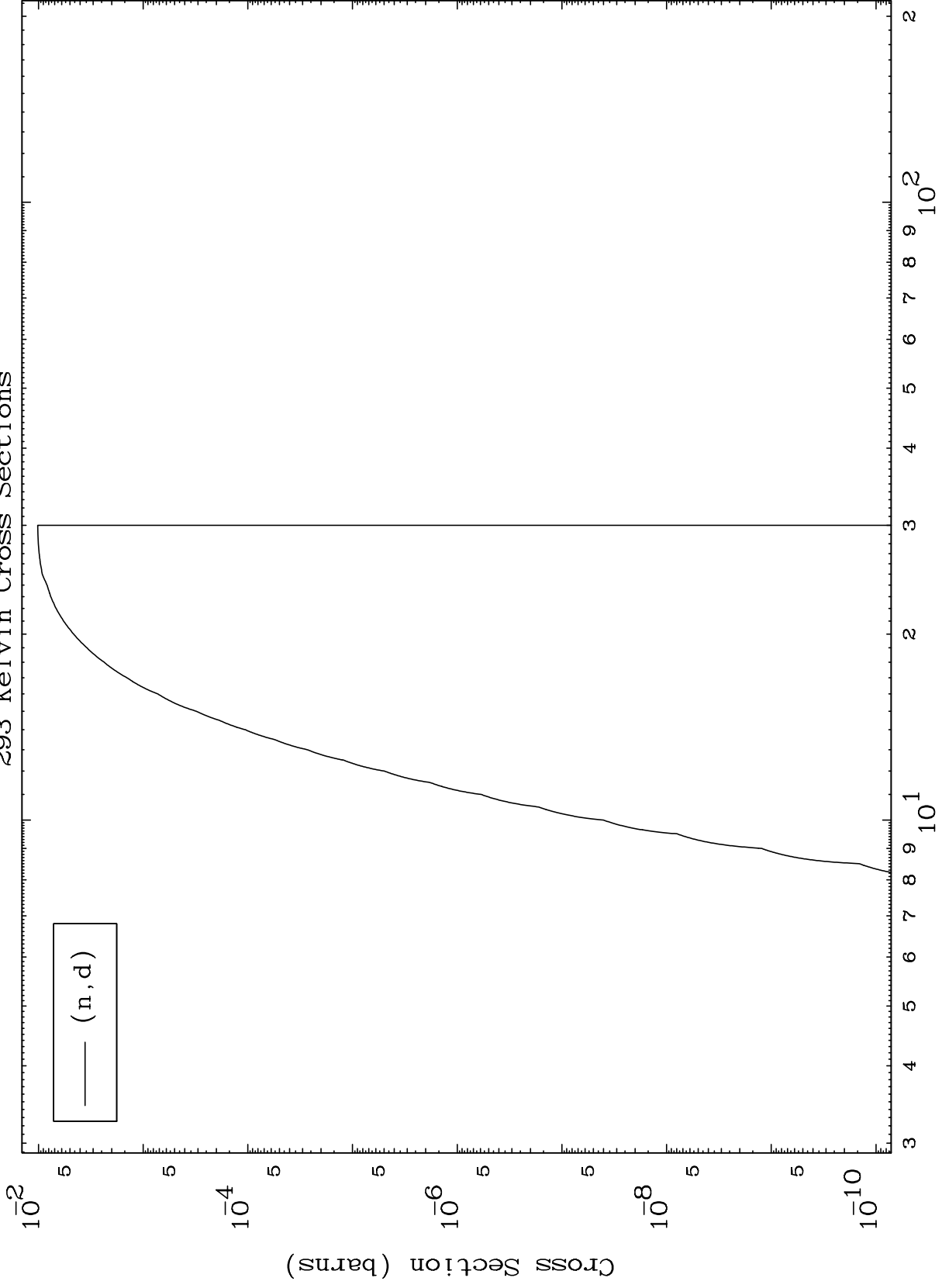
87-Fr-223



MAT 8758

(n,d) Levels
293 Kelvin Cross Sections

87-Fr-223



12

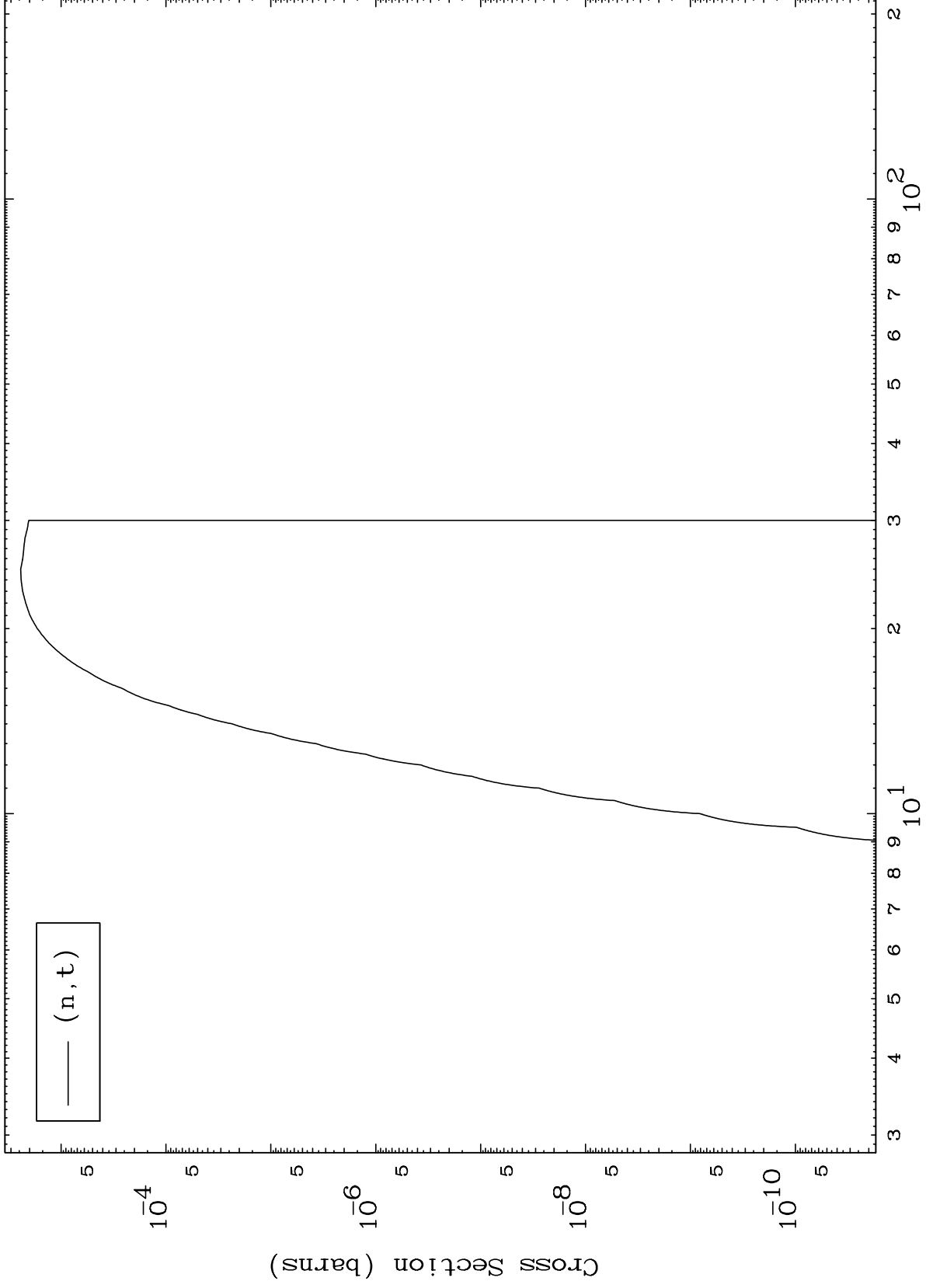
Incident Energy (MeV)

87-Fr-223

MAT 8758

(n,t) Levels
293 Kelvin Cross Sections

87-Fr-223



13

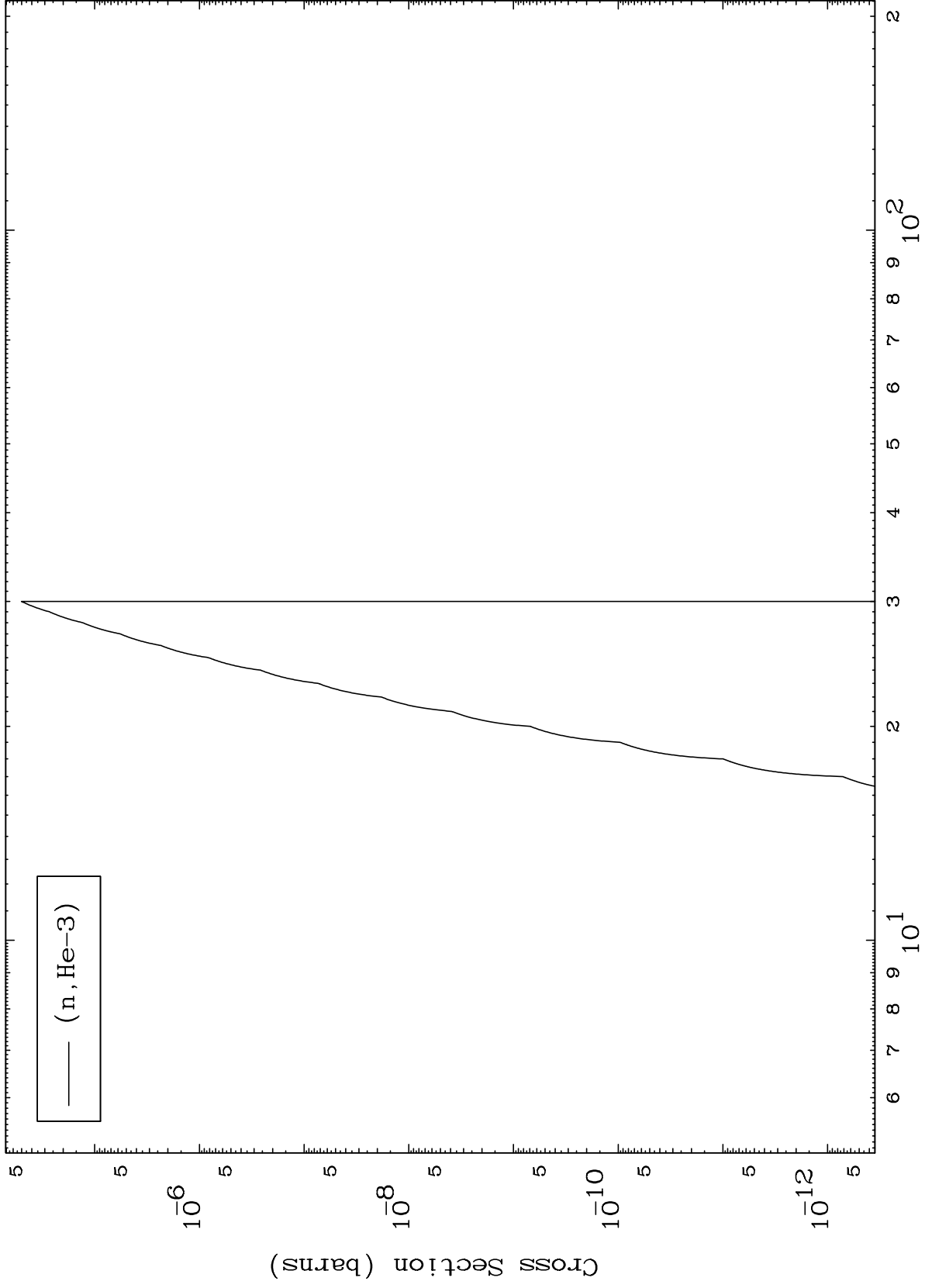
Incident Energy (MeV)

87-Fr-223

MAT 8758

(n,He3) Levels
293 Kelvin Cross Sections

87-Fr-223



14

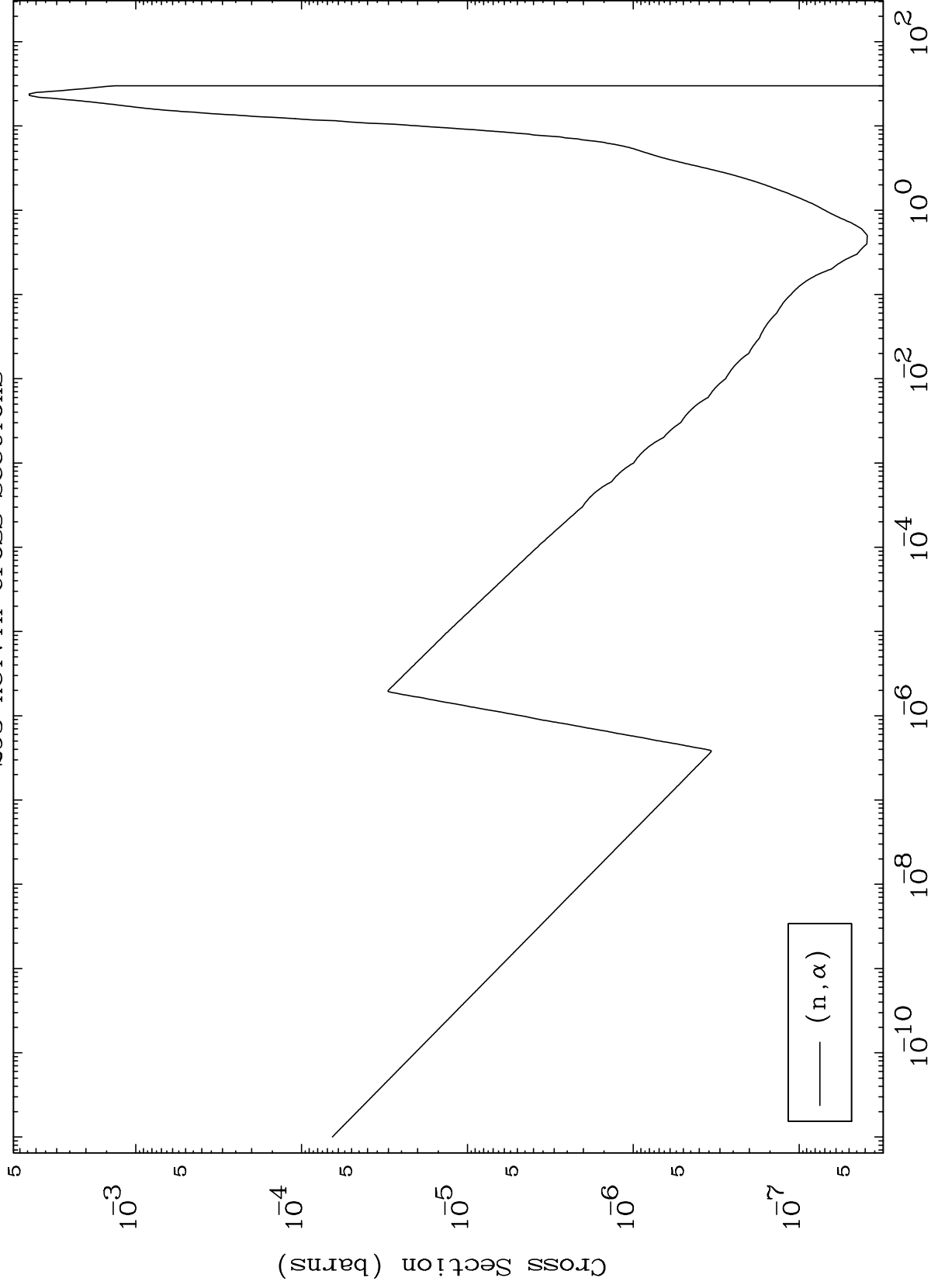
Incident Energy (MeV)

87-Fr-223

MAT 8758

(n, α) Levels
293 Kelvin Cross Sections

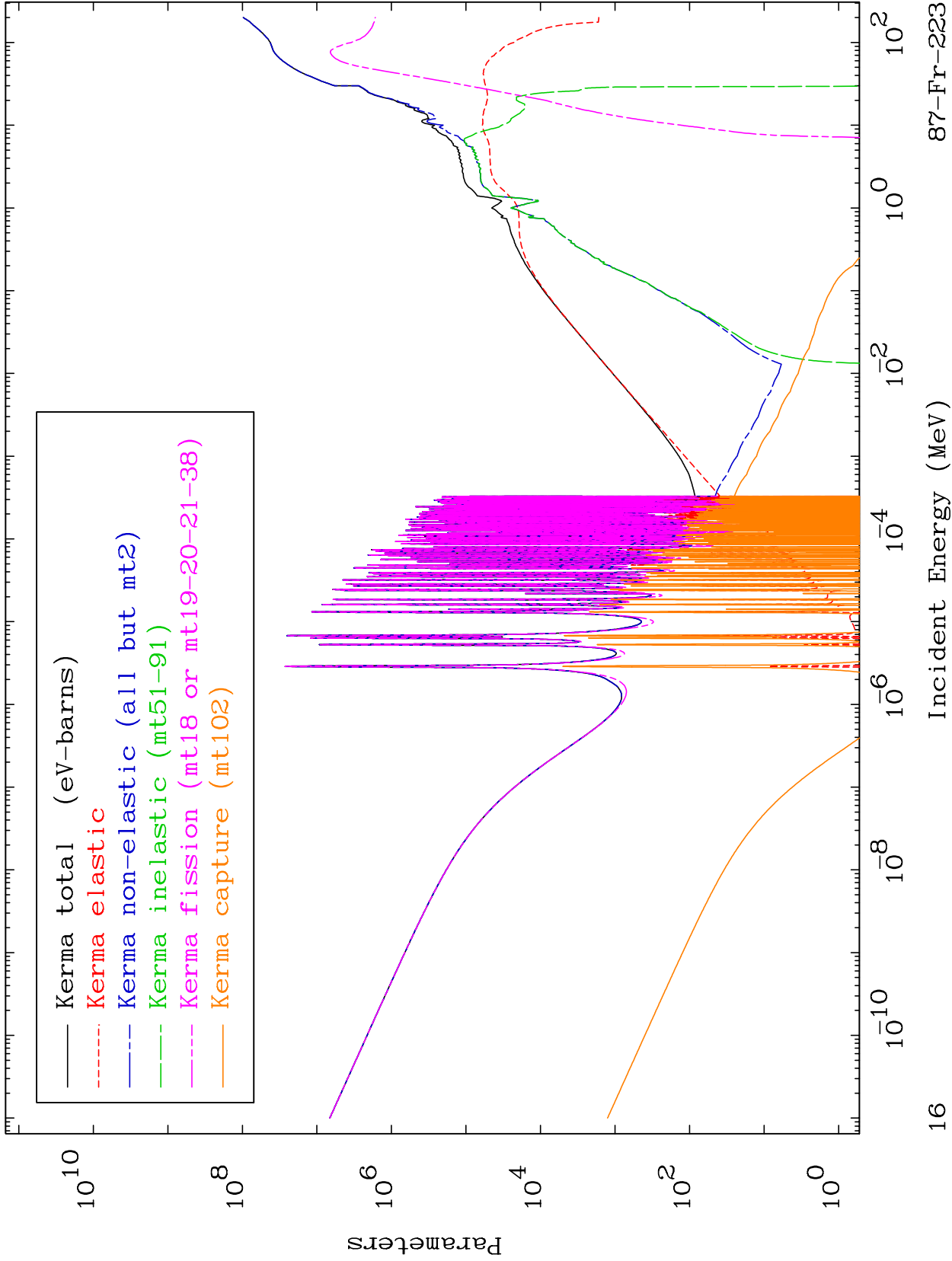
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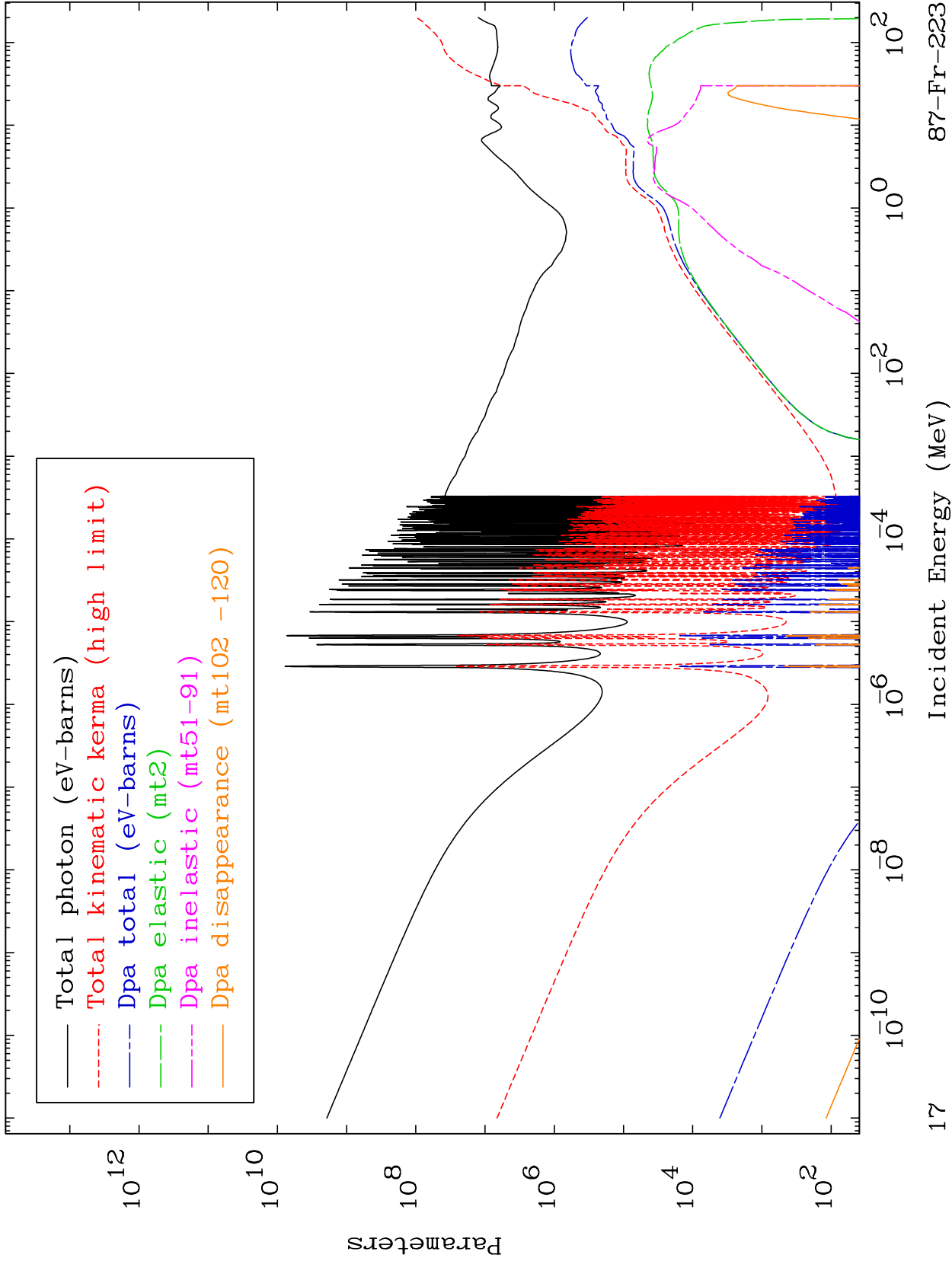


87-Fr-223

Incident Energy (MeV)

15

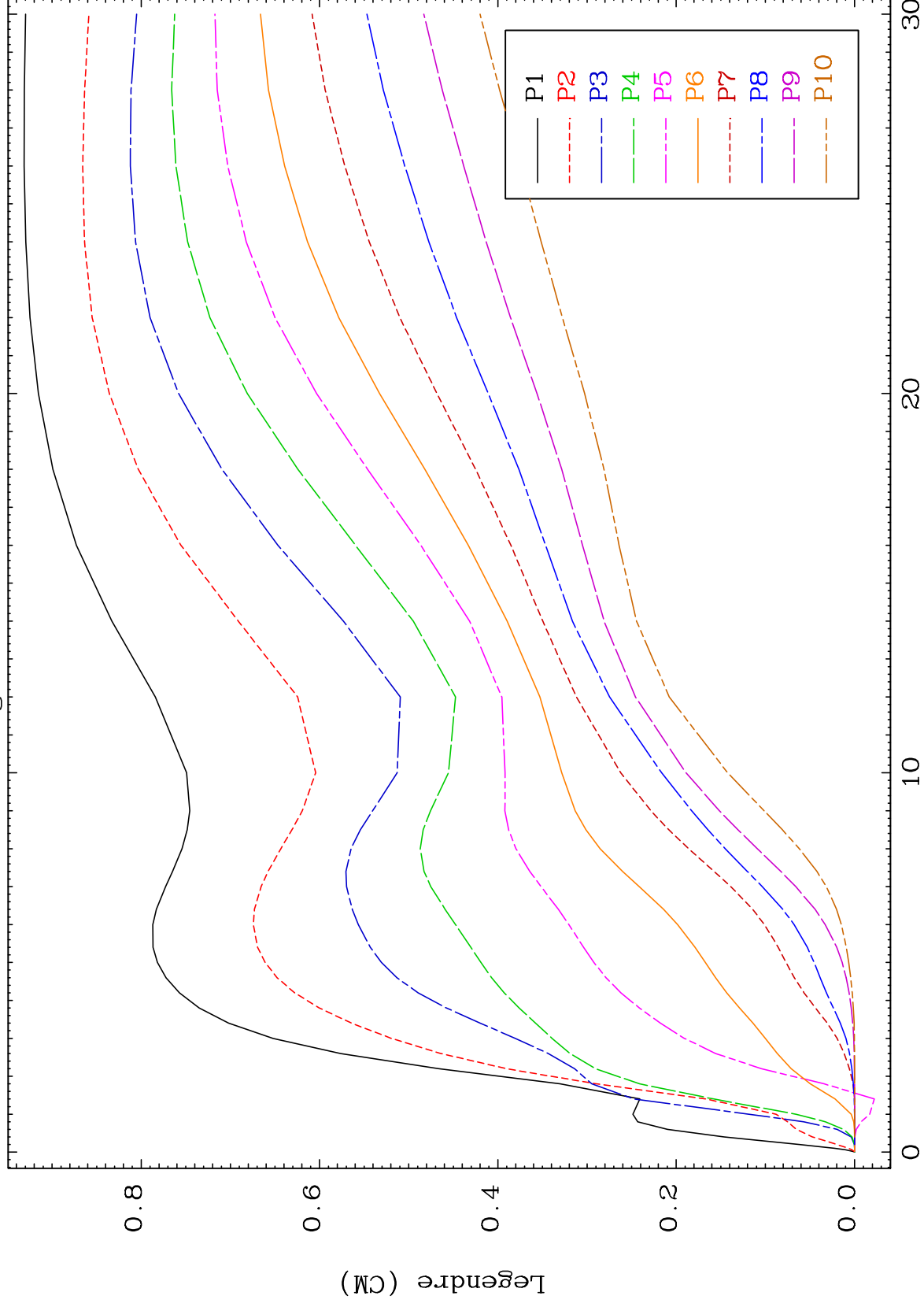




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Elastic Legendre Coefficients

87-Fr-223



18

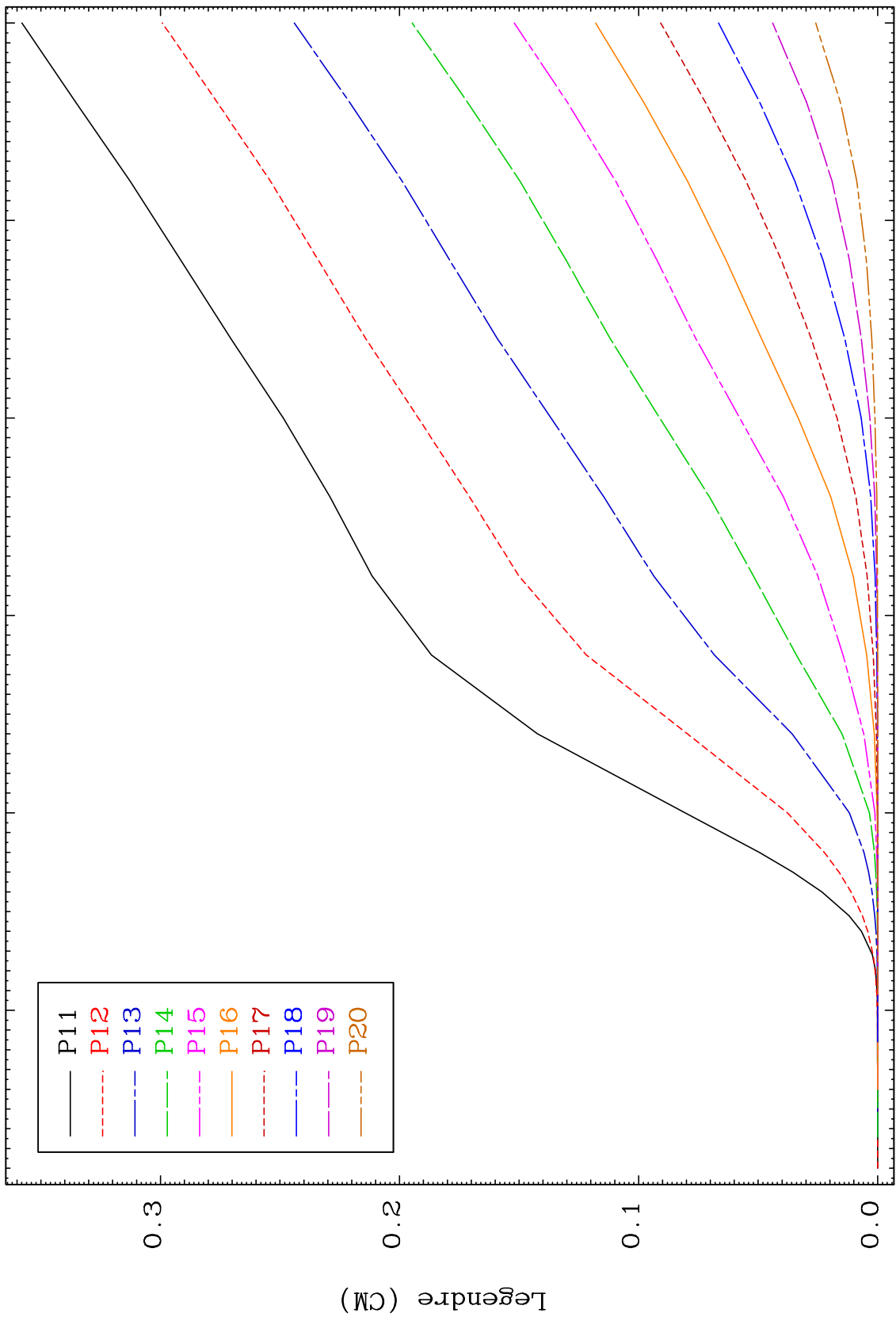
Incident Energy (MeV)

87-Fr-223

MAT 8758

Elastic Legendre Coefficients

87-Fr-223



19

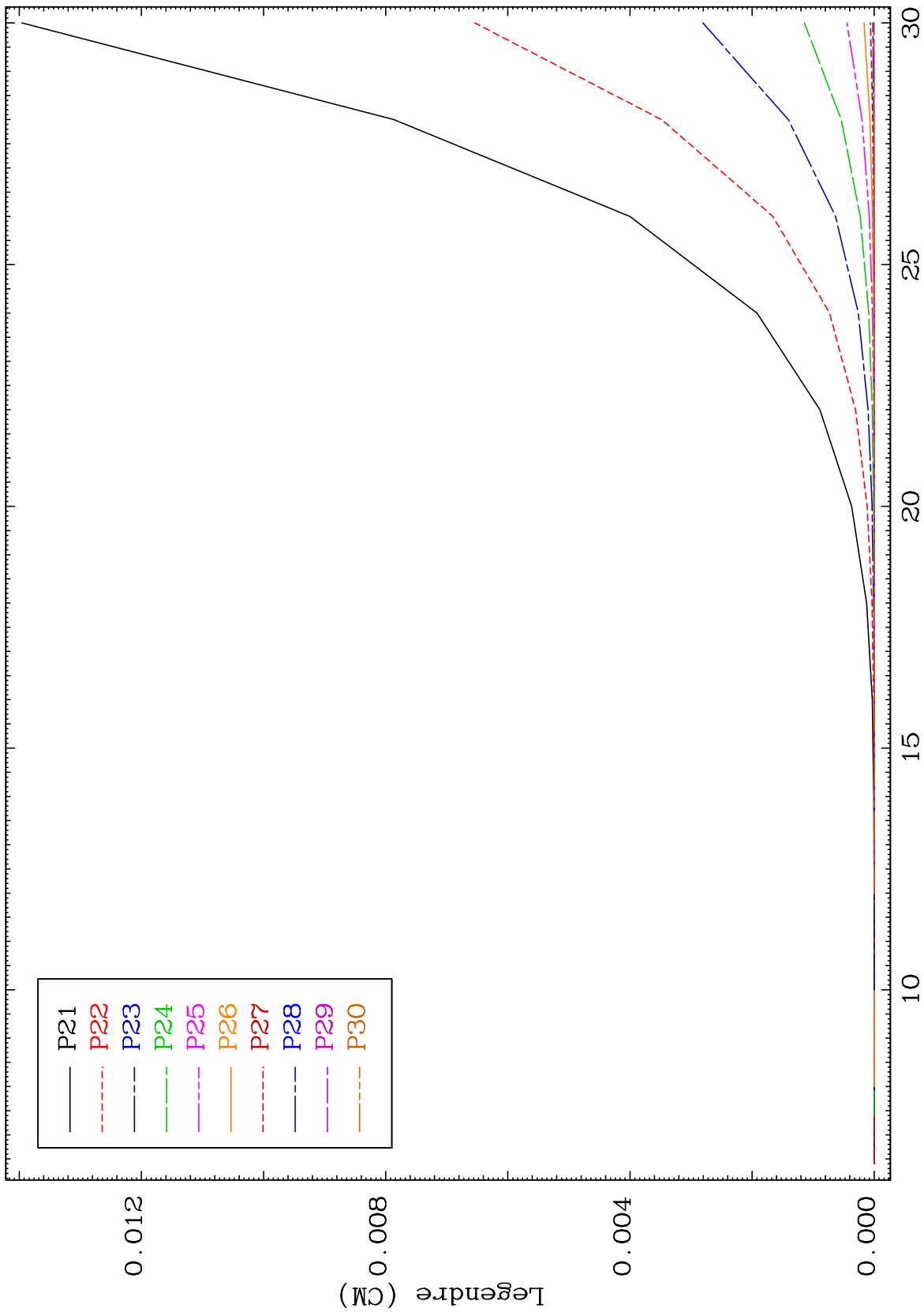
Incident Energy (MeV)

87-Fr-223

MAT 8758

Elastic
Legendre Coefficients

87-Fr-223



20

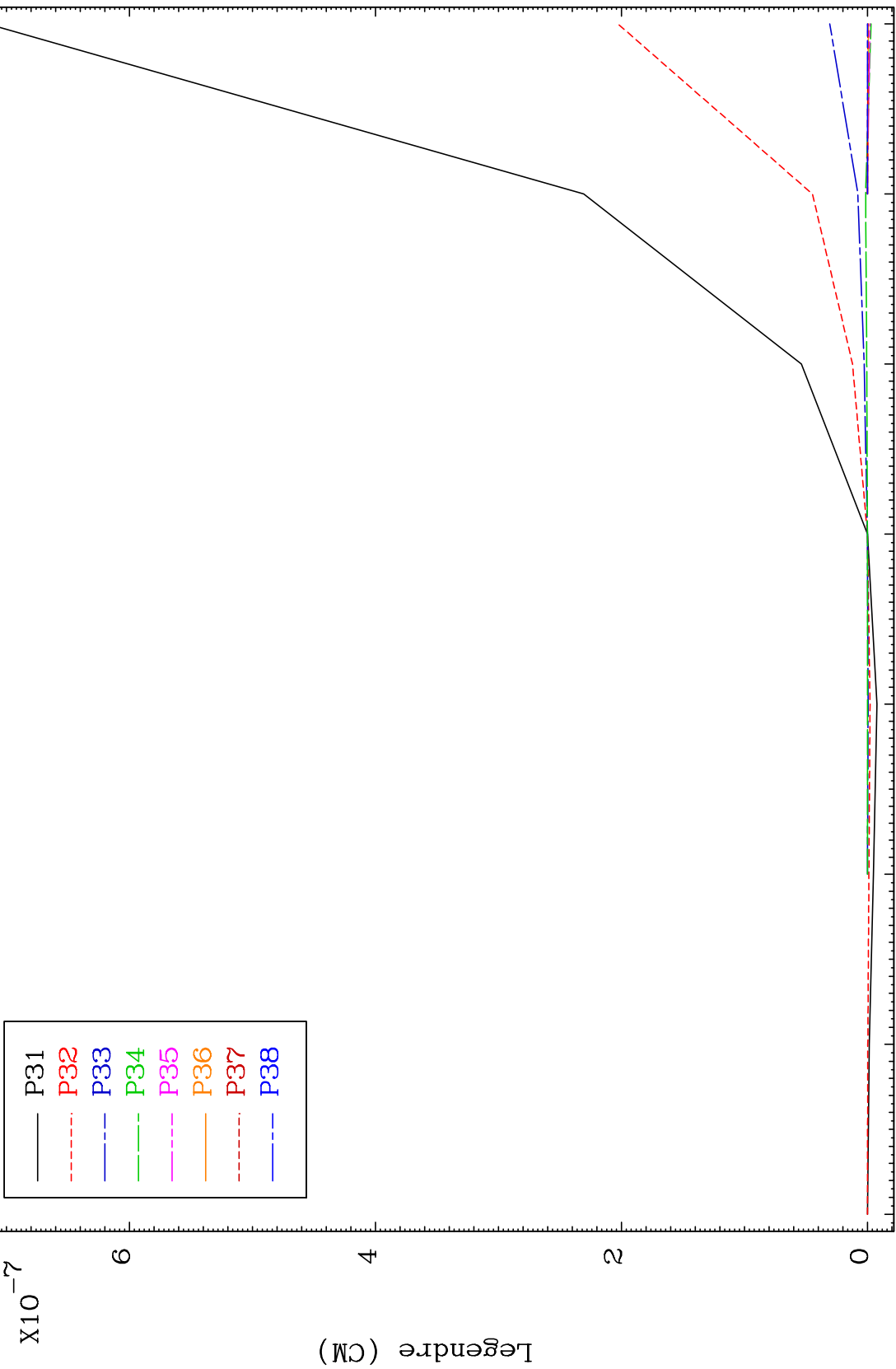
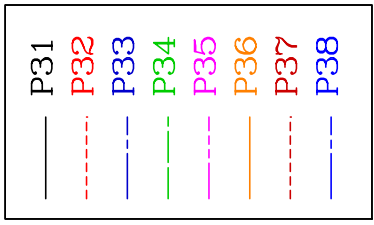
Incident Energy (MeV)

87-Fr-223

MAT 8758

Elastic Legendre Coefficients

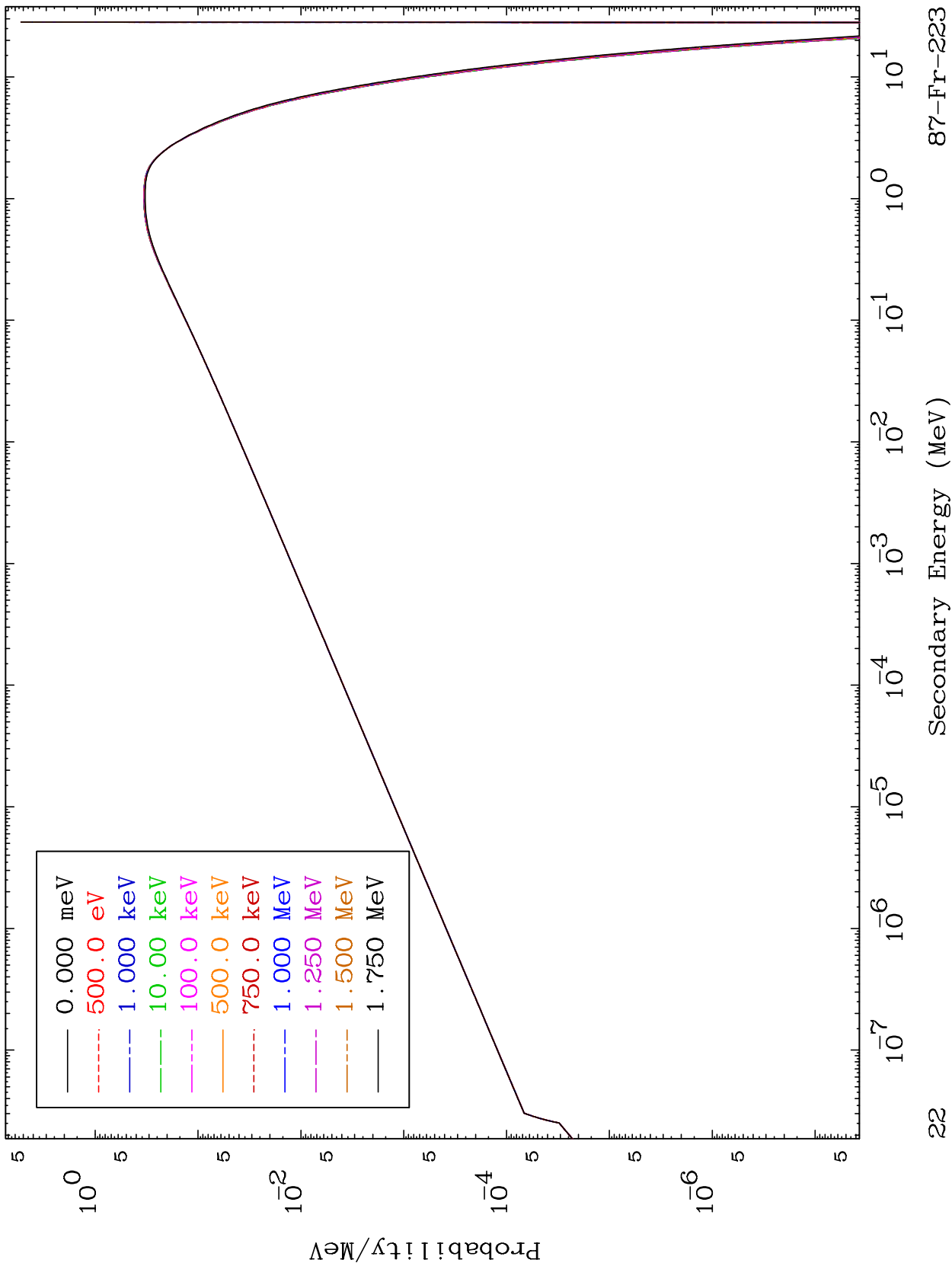
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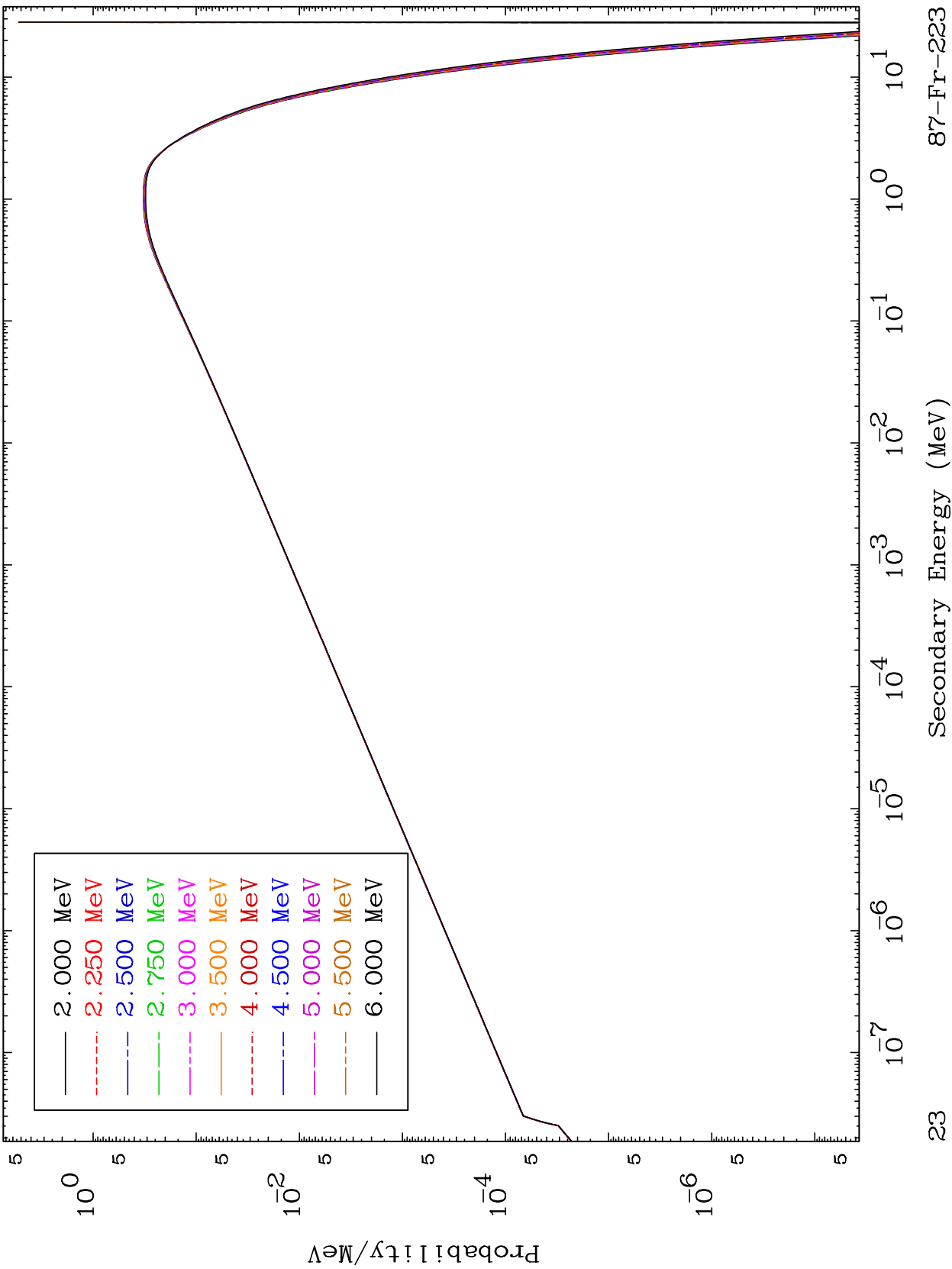


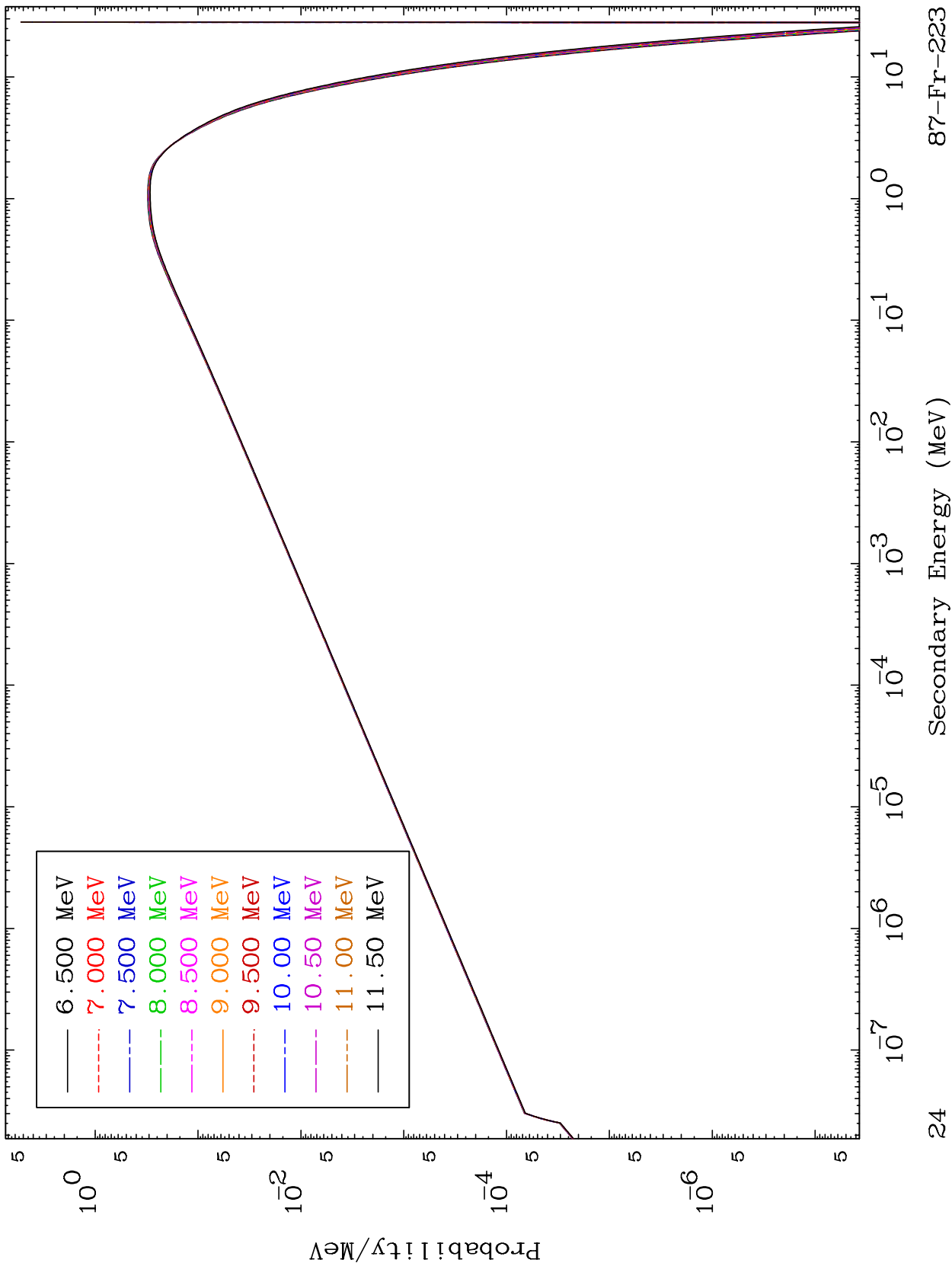
21

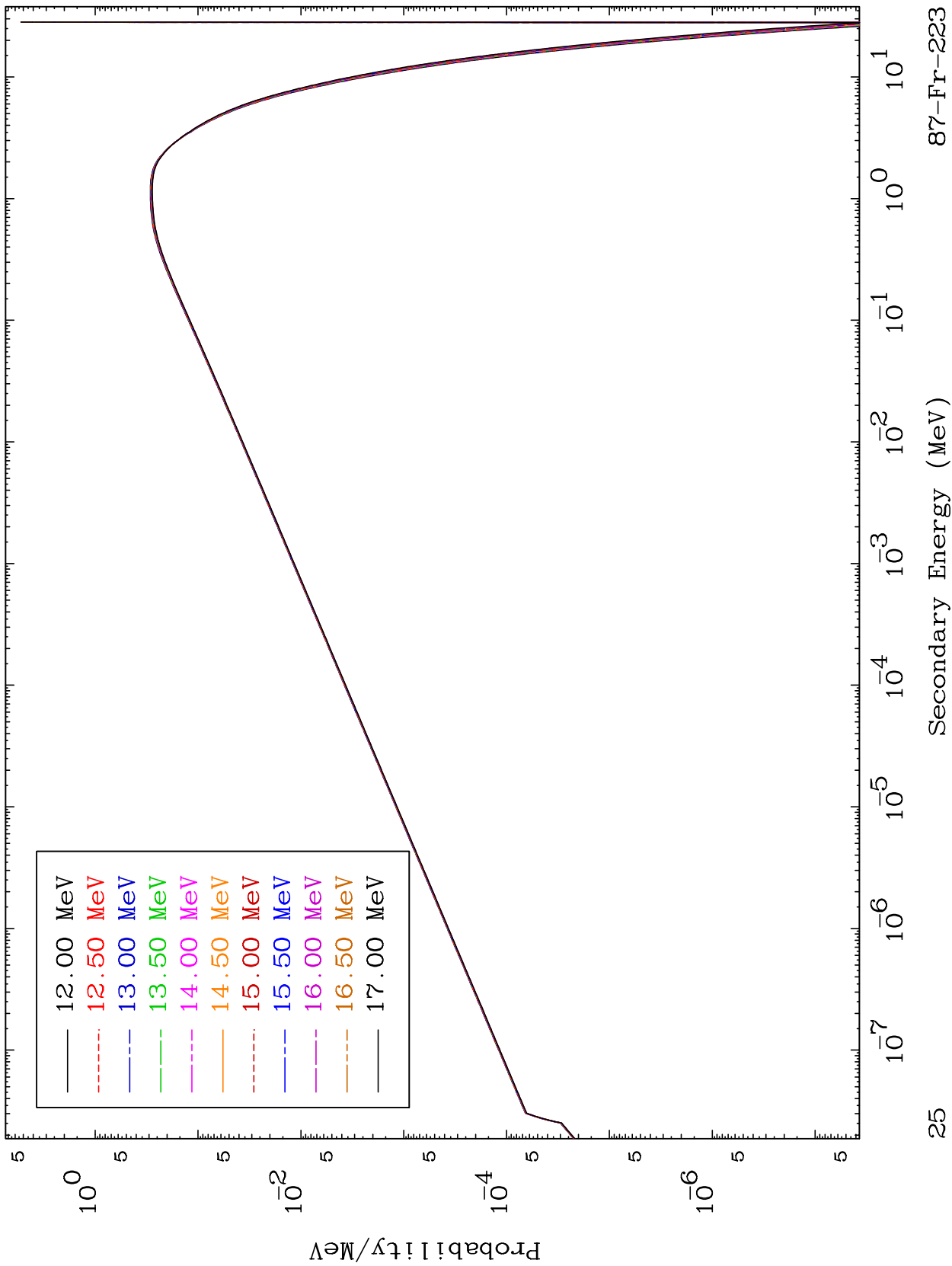
Incident Energy (MeV)

87-Fr-223

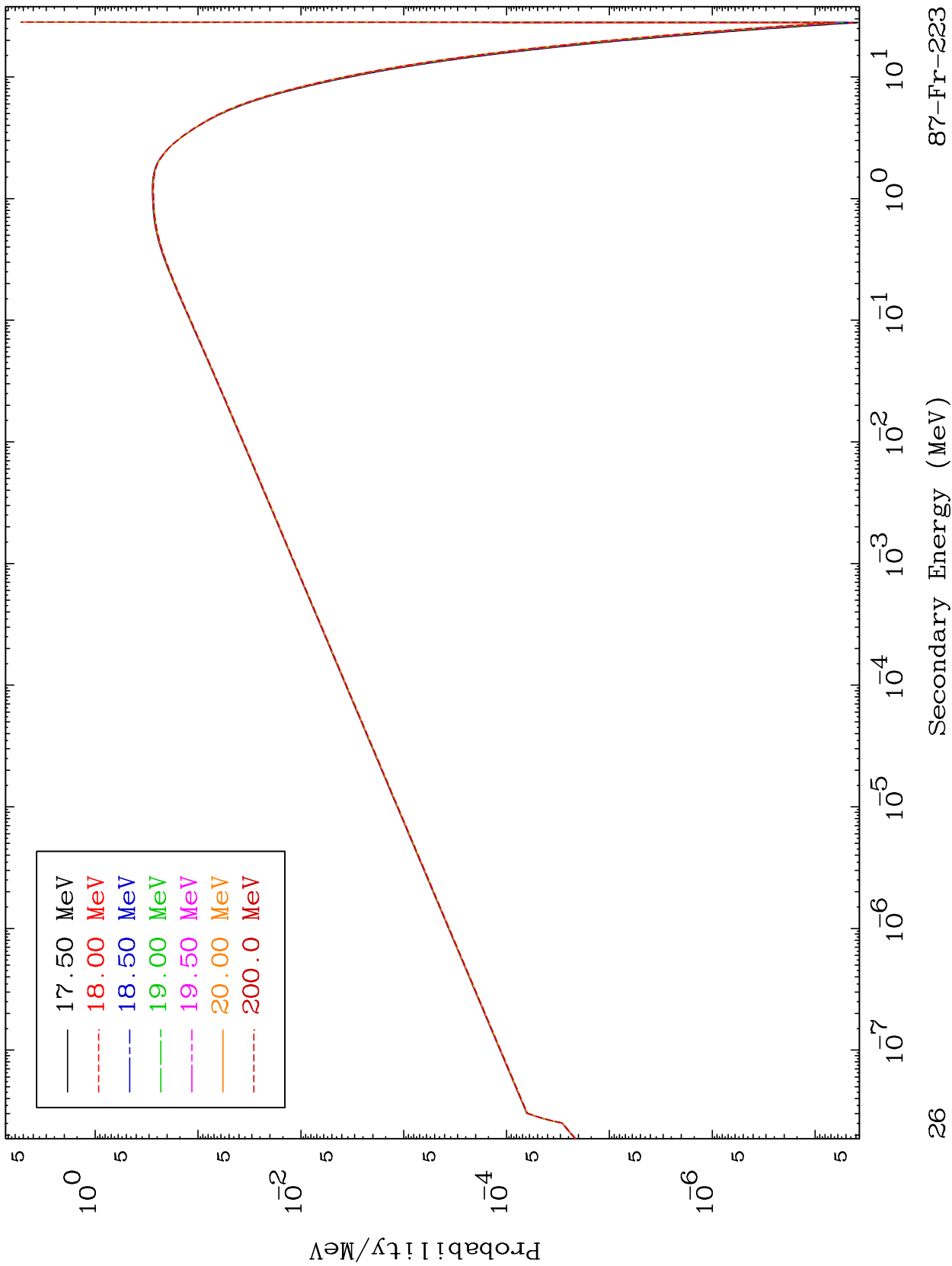








Fission
Energy Distribution

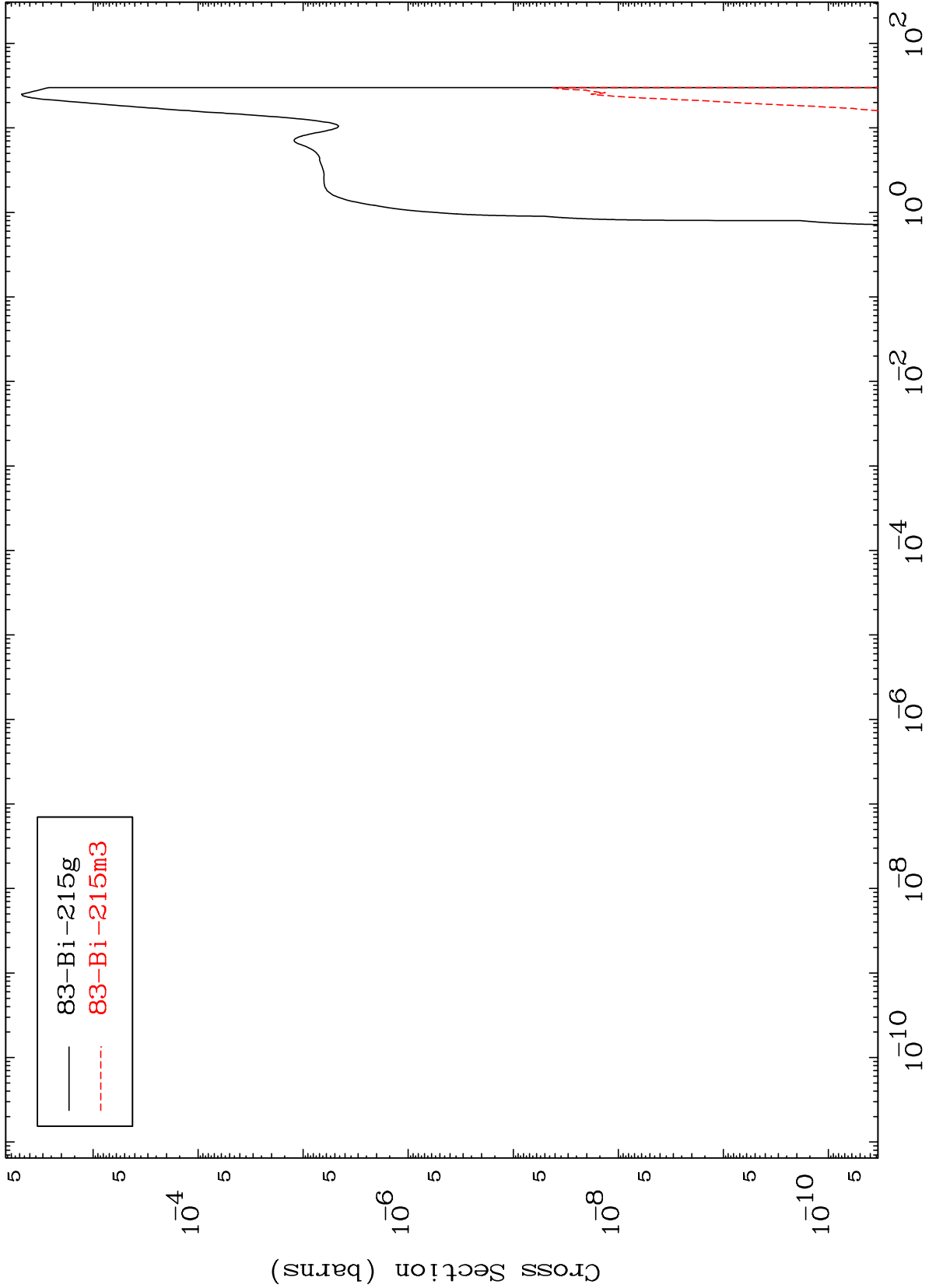


MAT 8758

(n,n') 2 α

87-Fr-223

Radionuclide Production Cross Section



83-Bi-215g
83-Bi-215m3

Incident Energy (MeV)

87-Fr-223

27

MAT 8758

(n,2α)

87-Fr-223

Radionuclide Production Cross Section

