

Program EVALPLOT  
(Version 2015-2)

by

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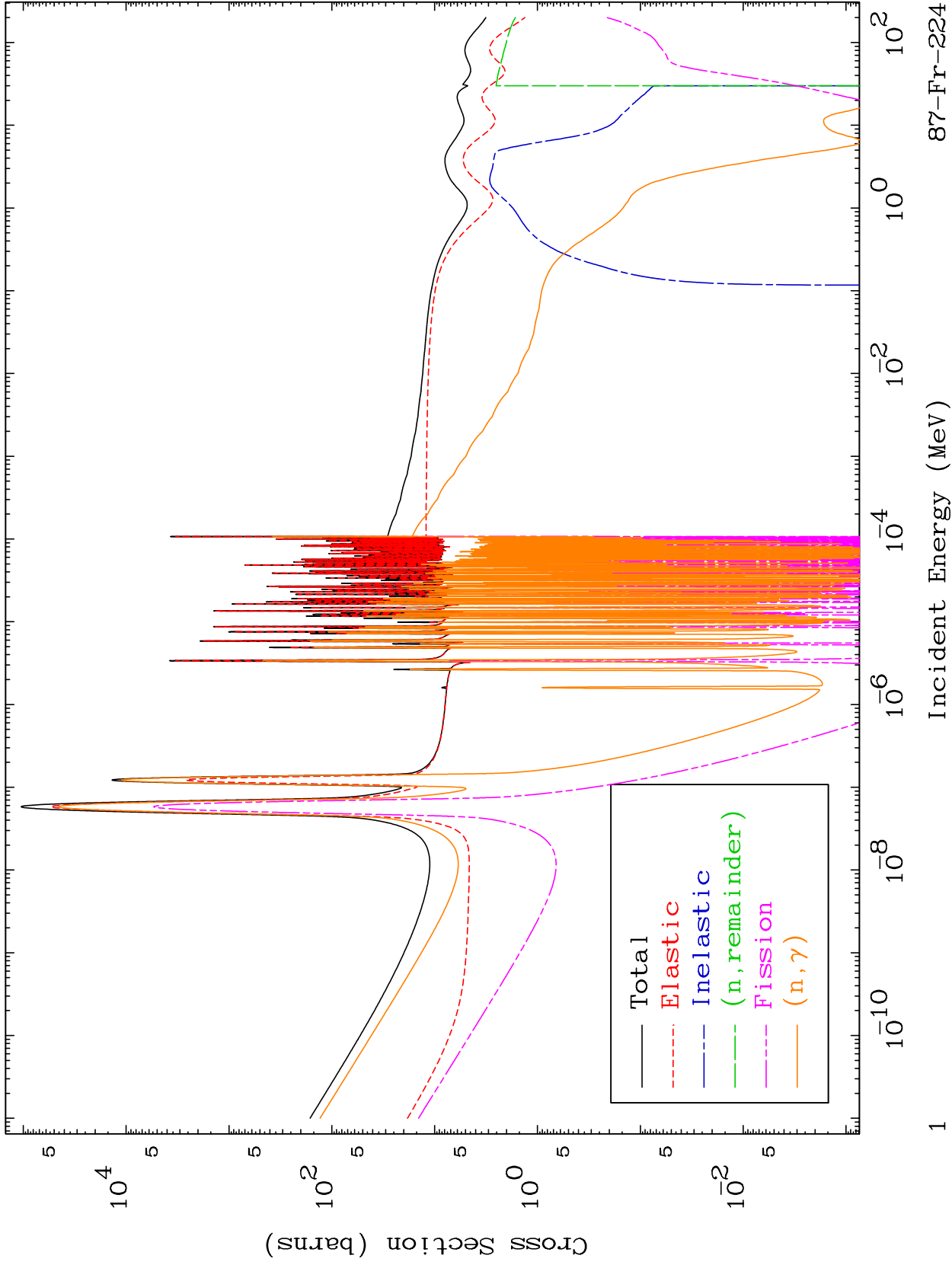
Press Mouse Button to Start

MAT 8761

Major

293 Kelvin Cross Sections

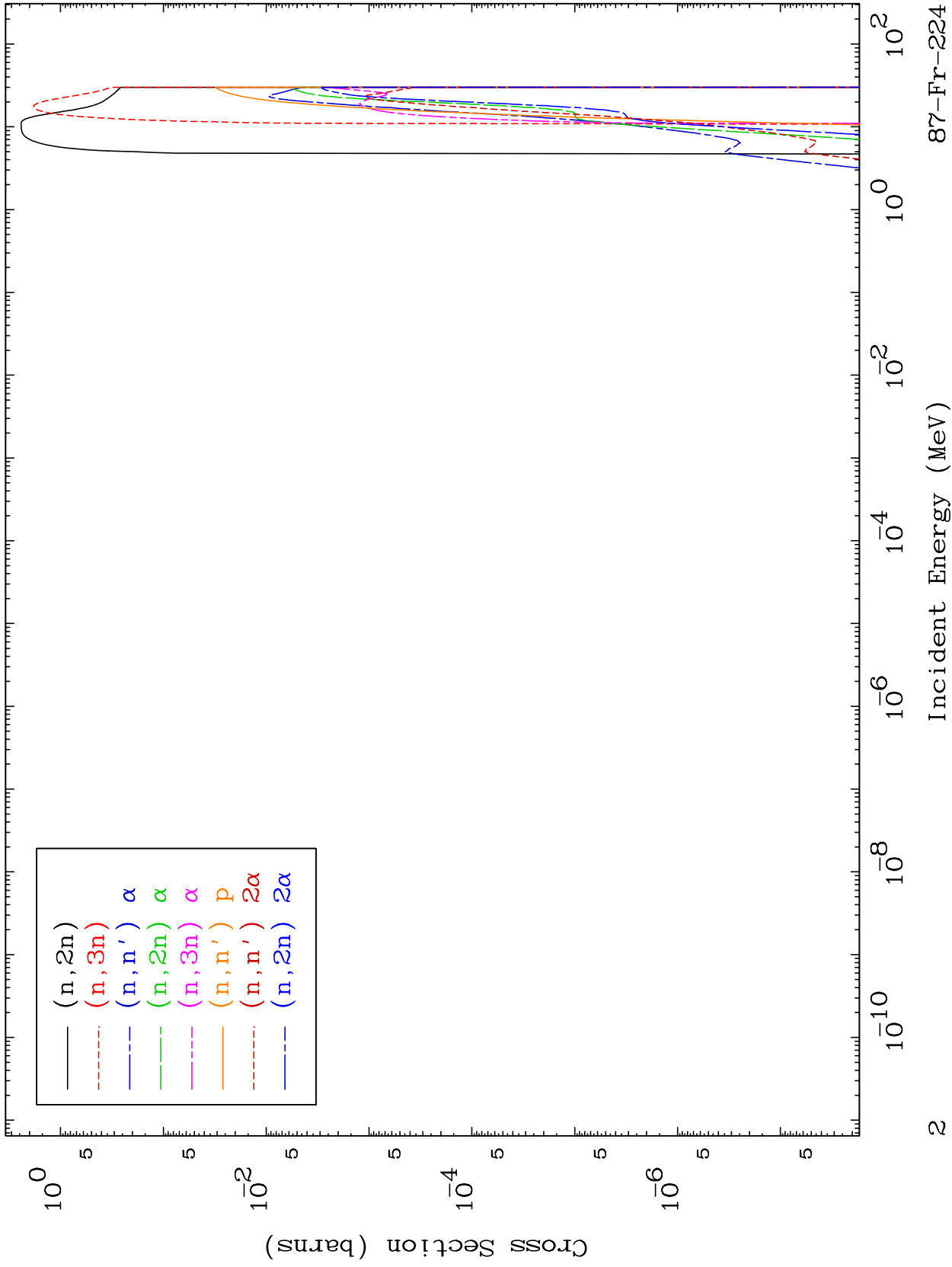
87-Fr-224



MAT 8761

Neutron Production  
293 Kelvin Cross Sections

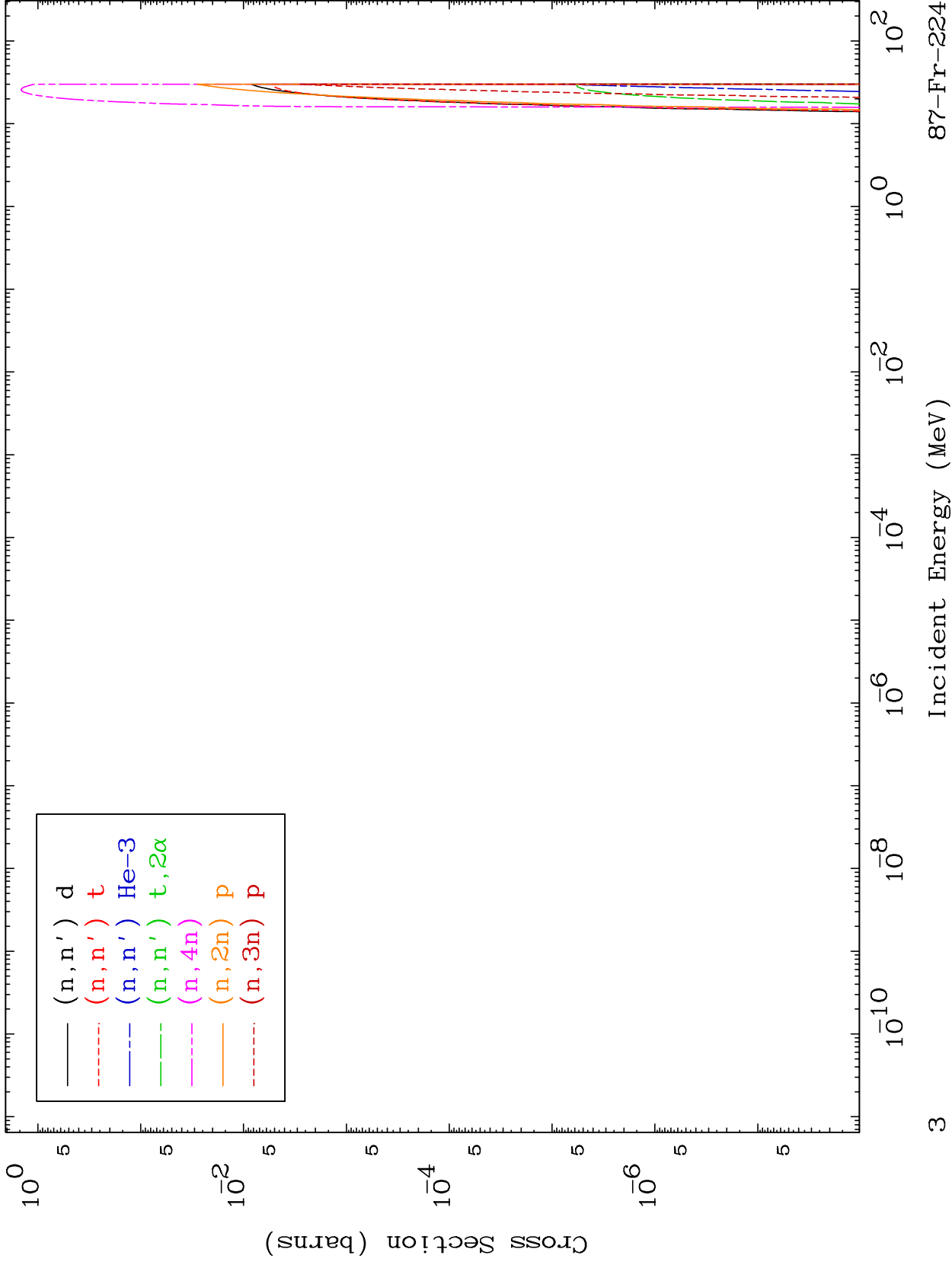
87-Fr-224



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Neutron Production  
293 Kelvin Cross Sections

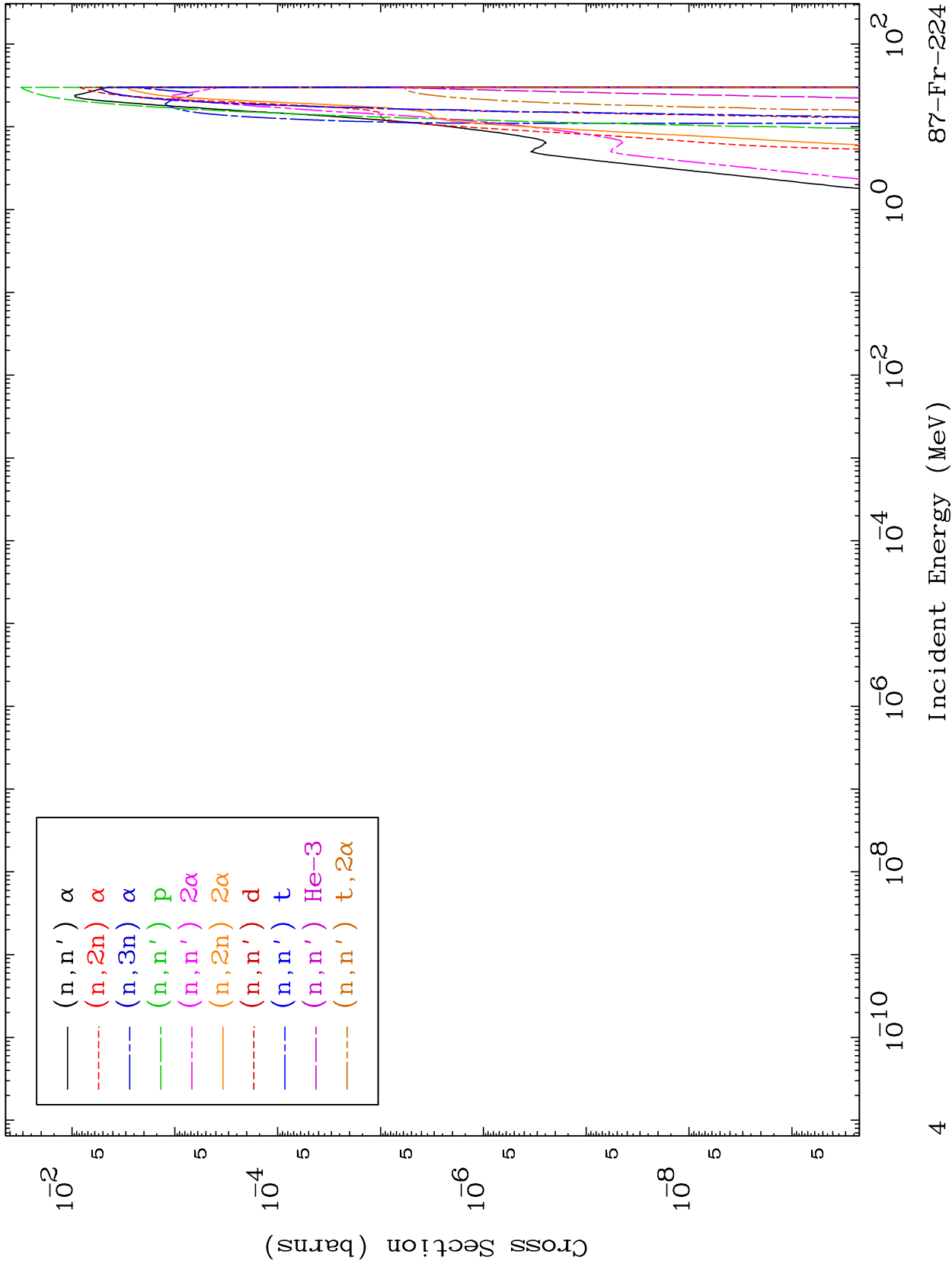
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Charged Particle  
293 Kelvin Cross Sections

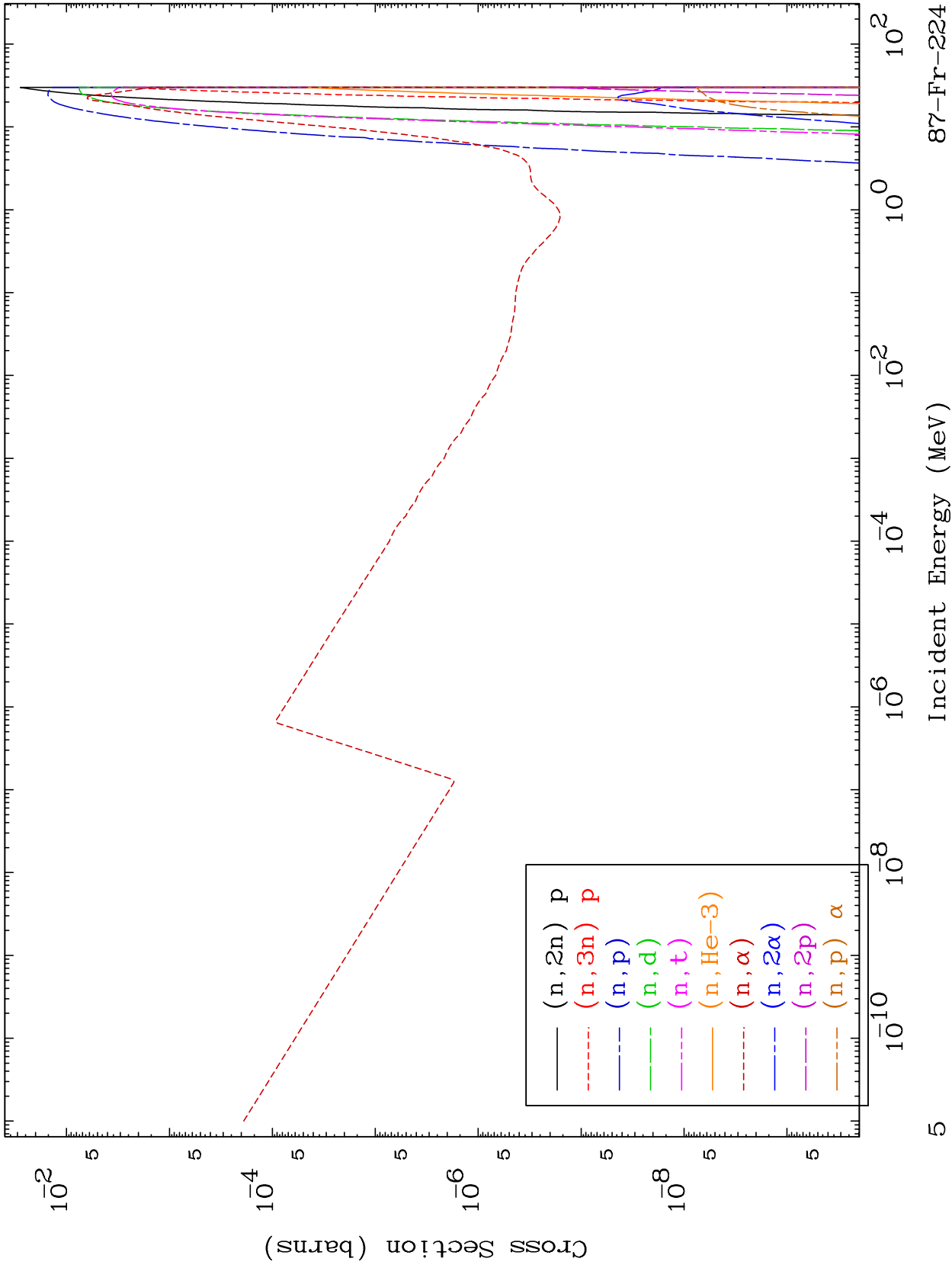
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Charged Particle  
293 Kelvin Cross Sections

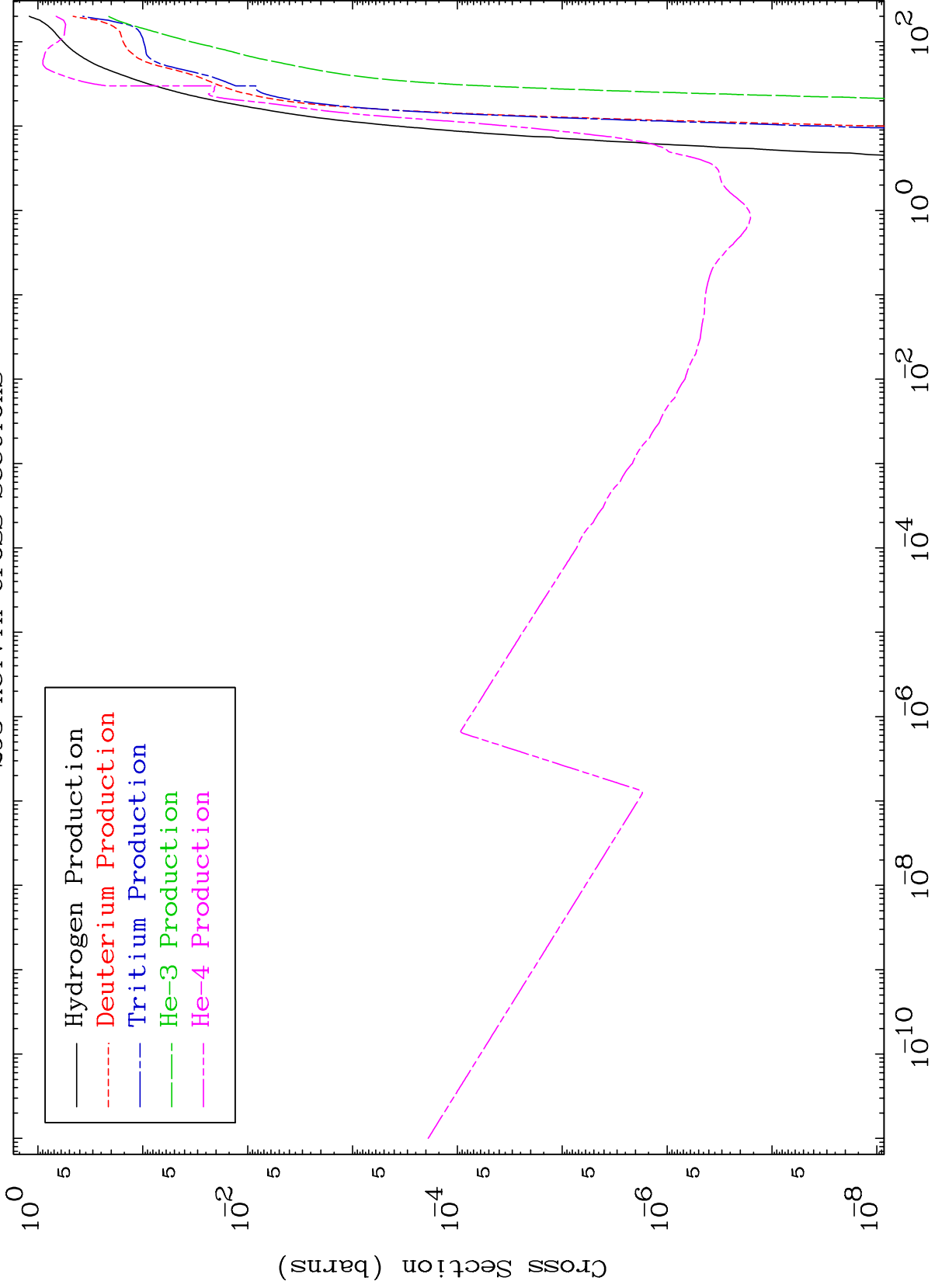
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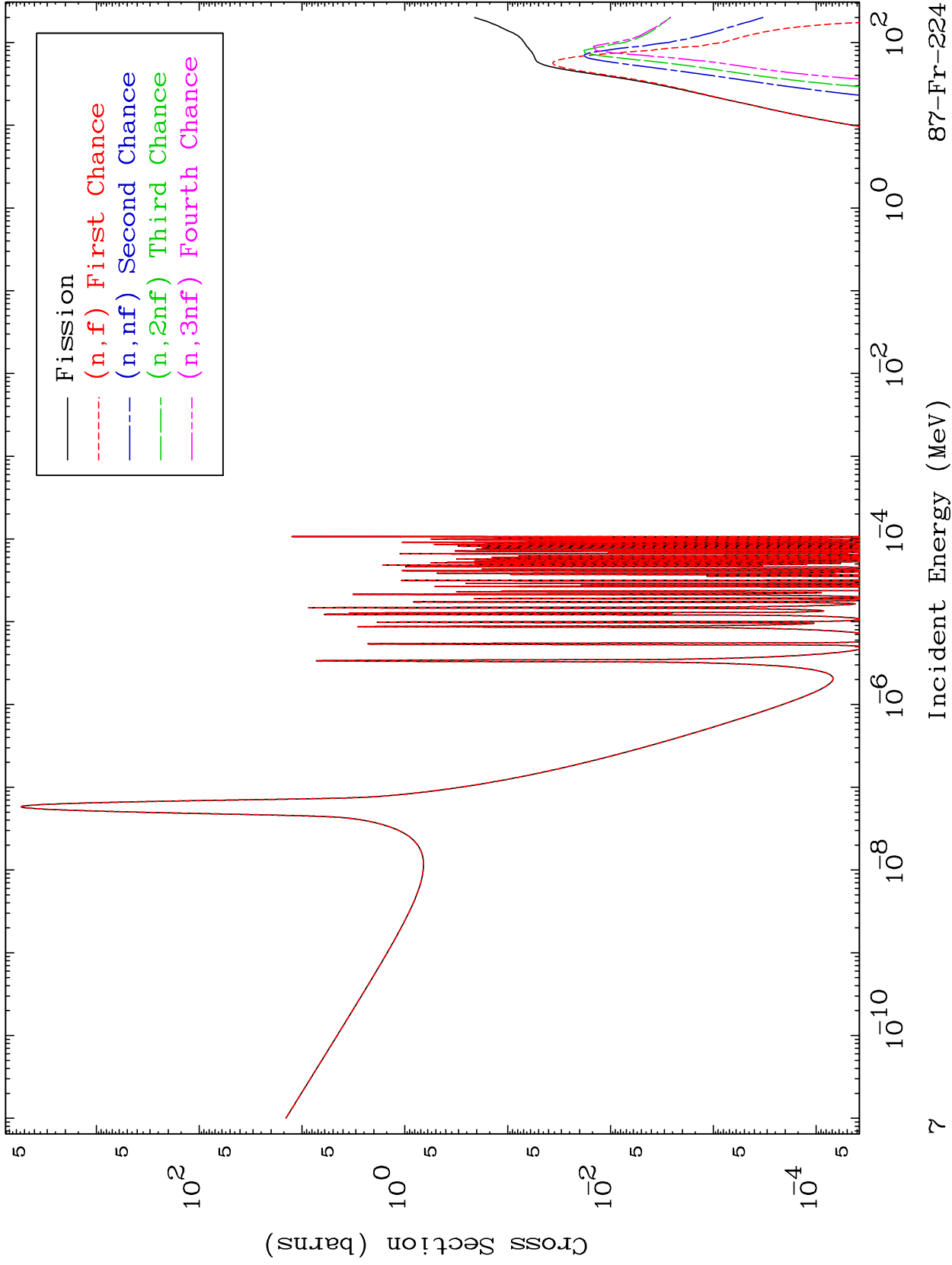


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Particle Production  
293 Kelvin Cross Sections

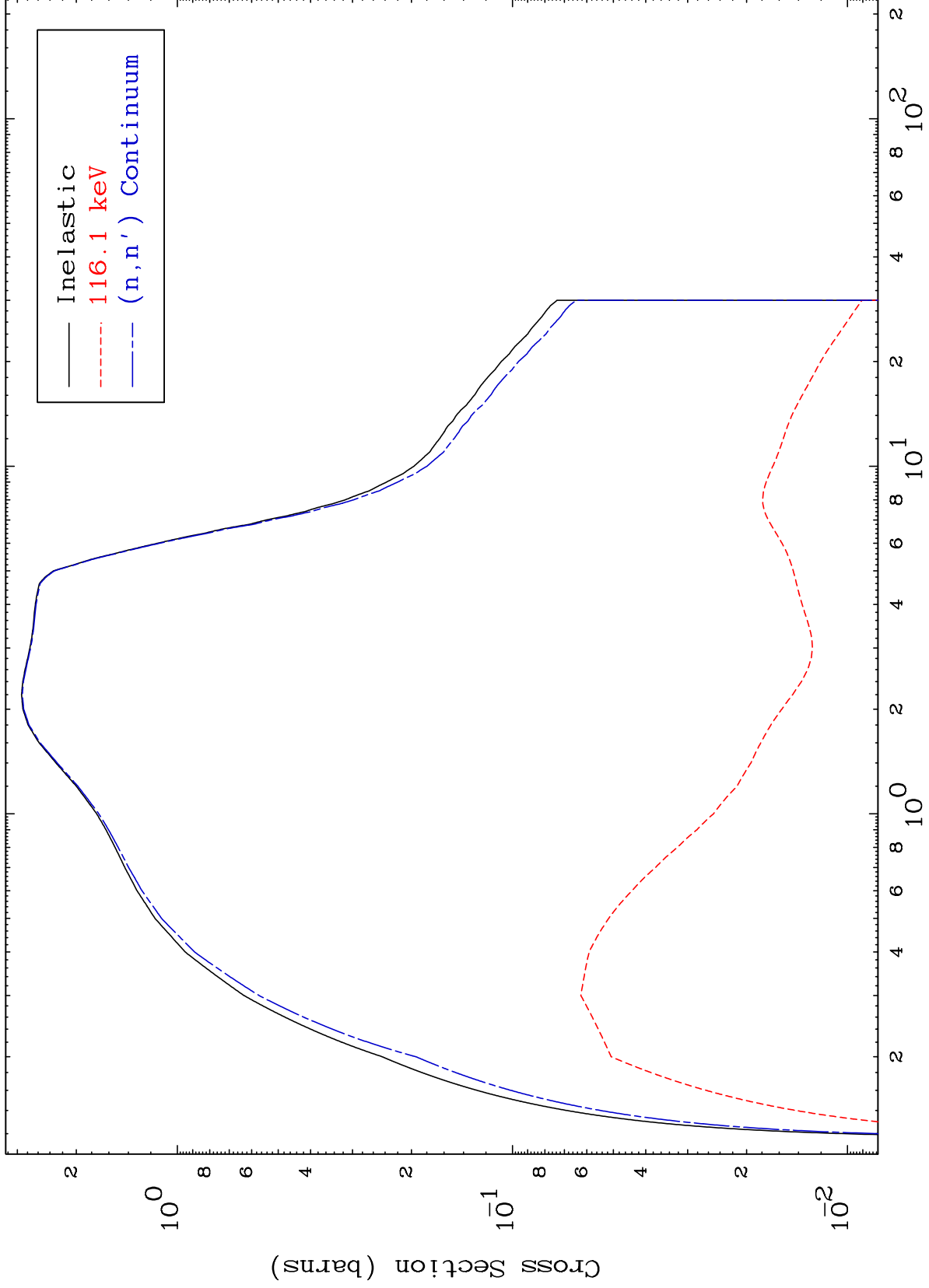
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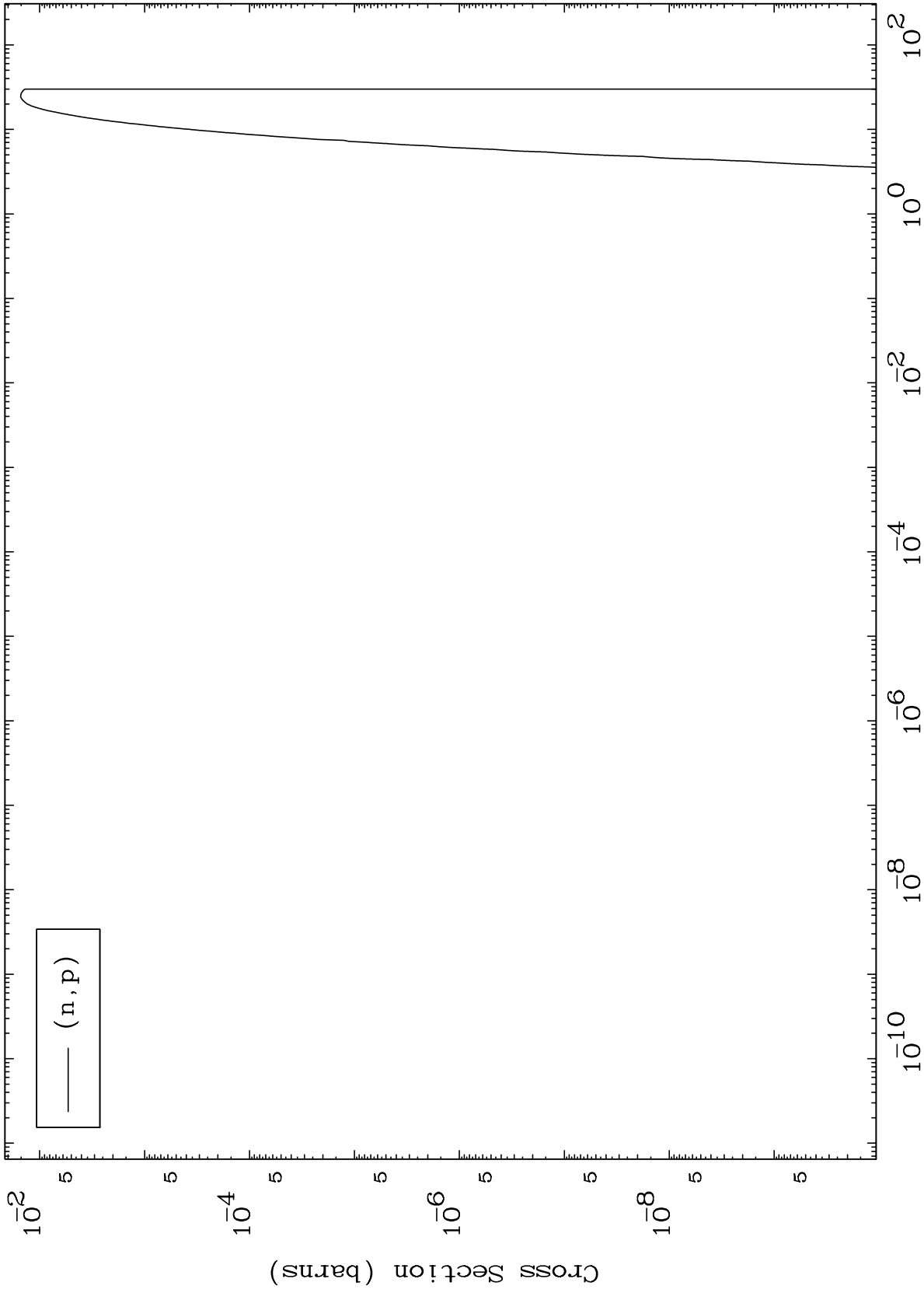
293 Kelvin Cross Sections



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(n,p) Levels  
293 Kelvin Cross Sections

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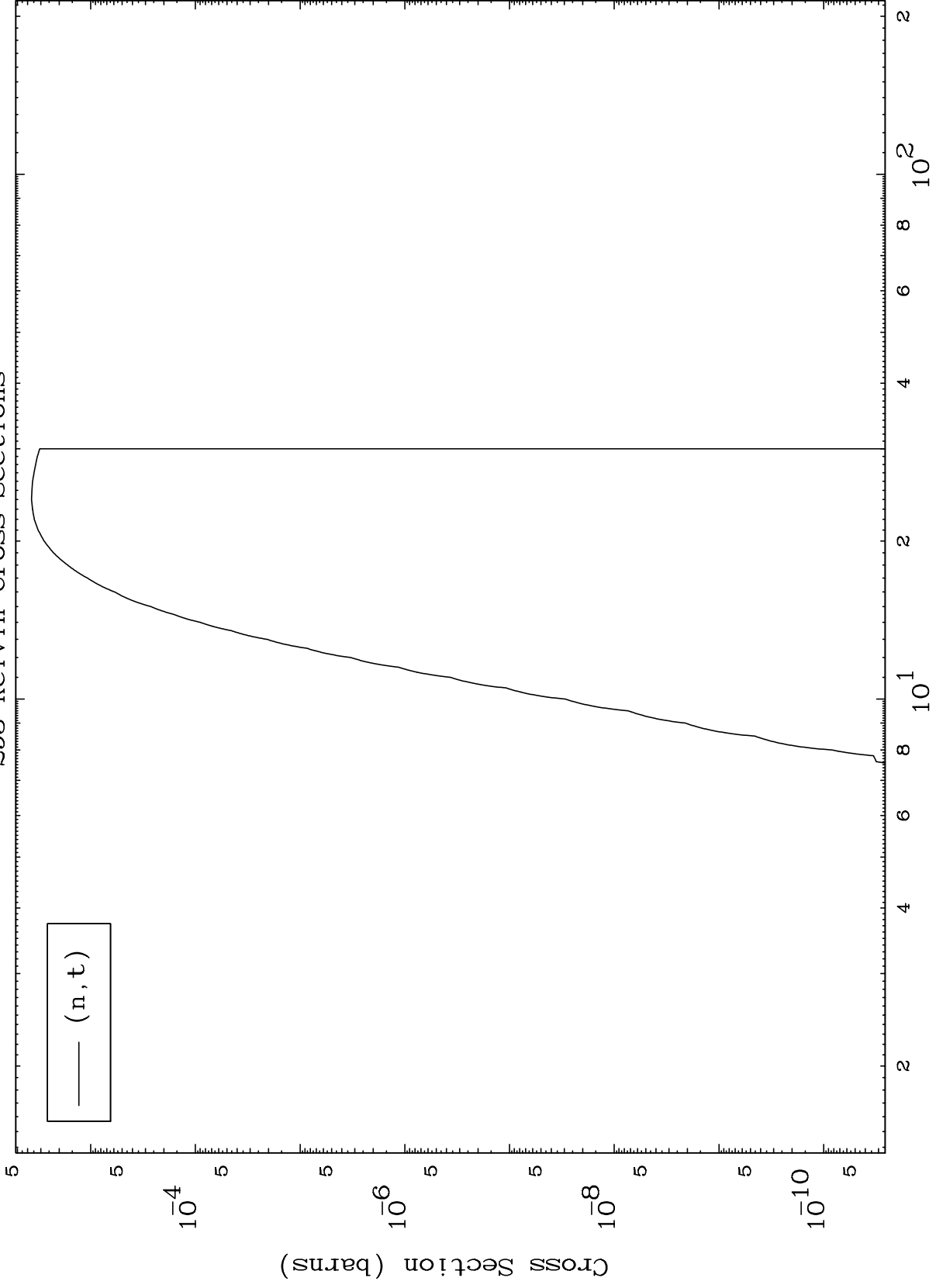




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(n,t) Levels  
293 Kelvin Cross Sections

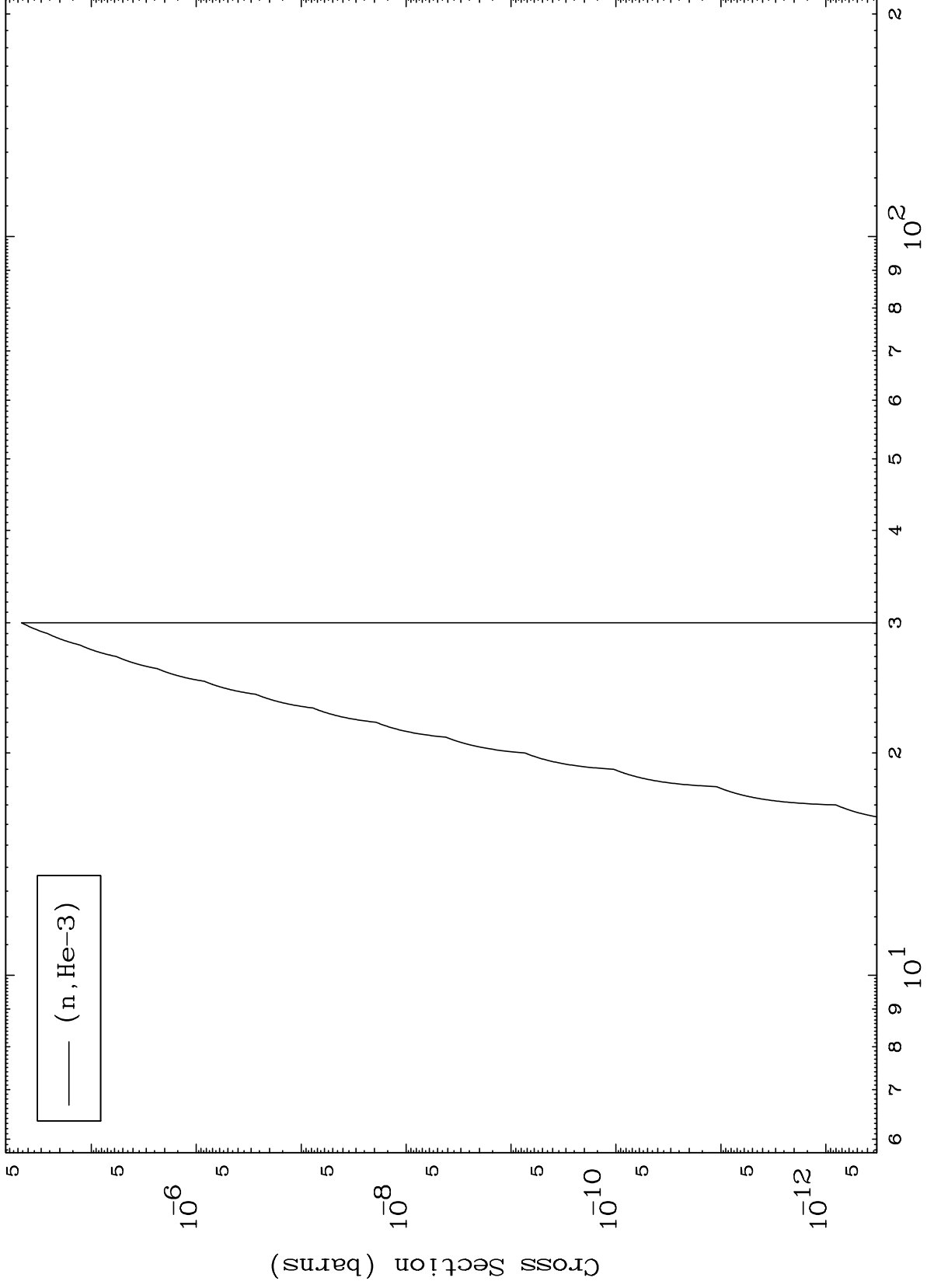
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(n,He3) Levels  
293 Kelvin Cross Sections

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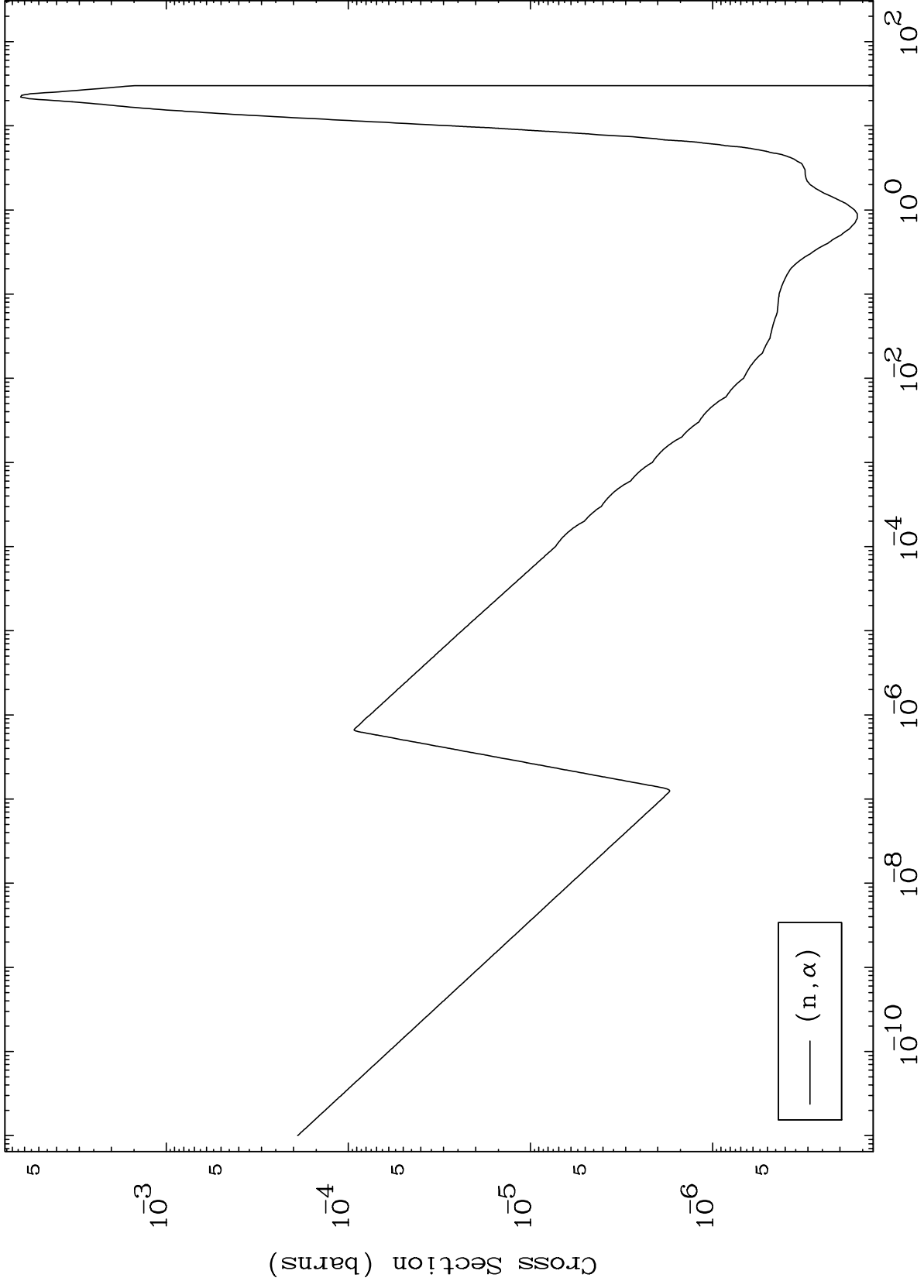
Incident Energy (MeV)

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(n,α) Levels  
293 Kelvin Cross Sections

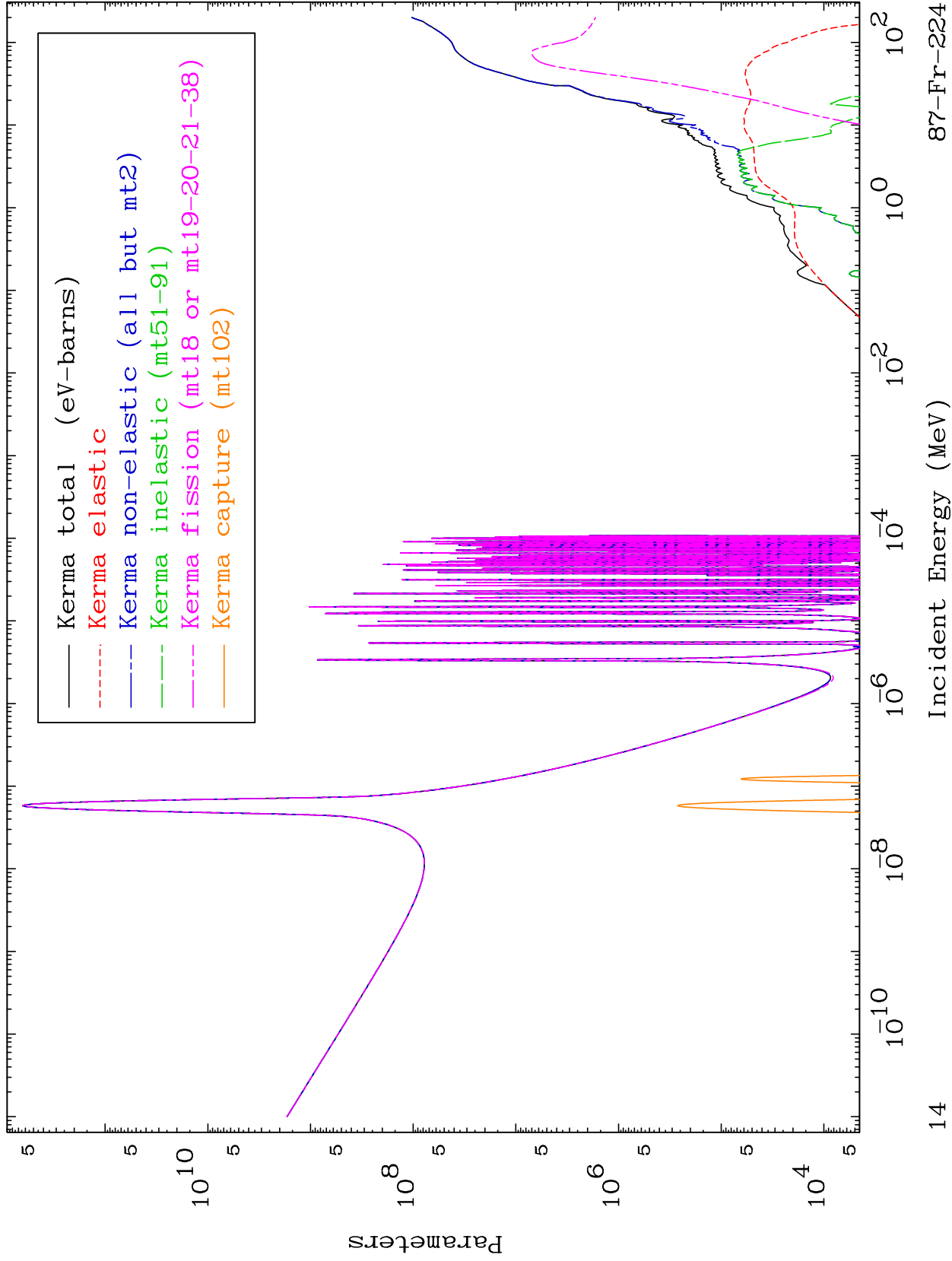
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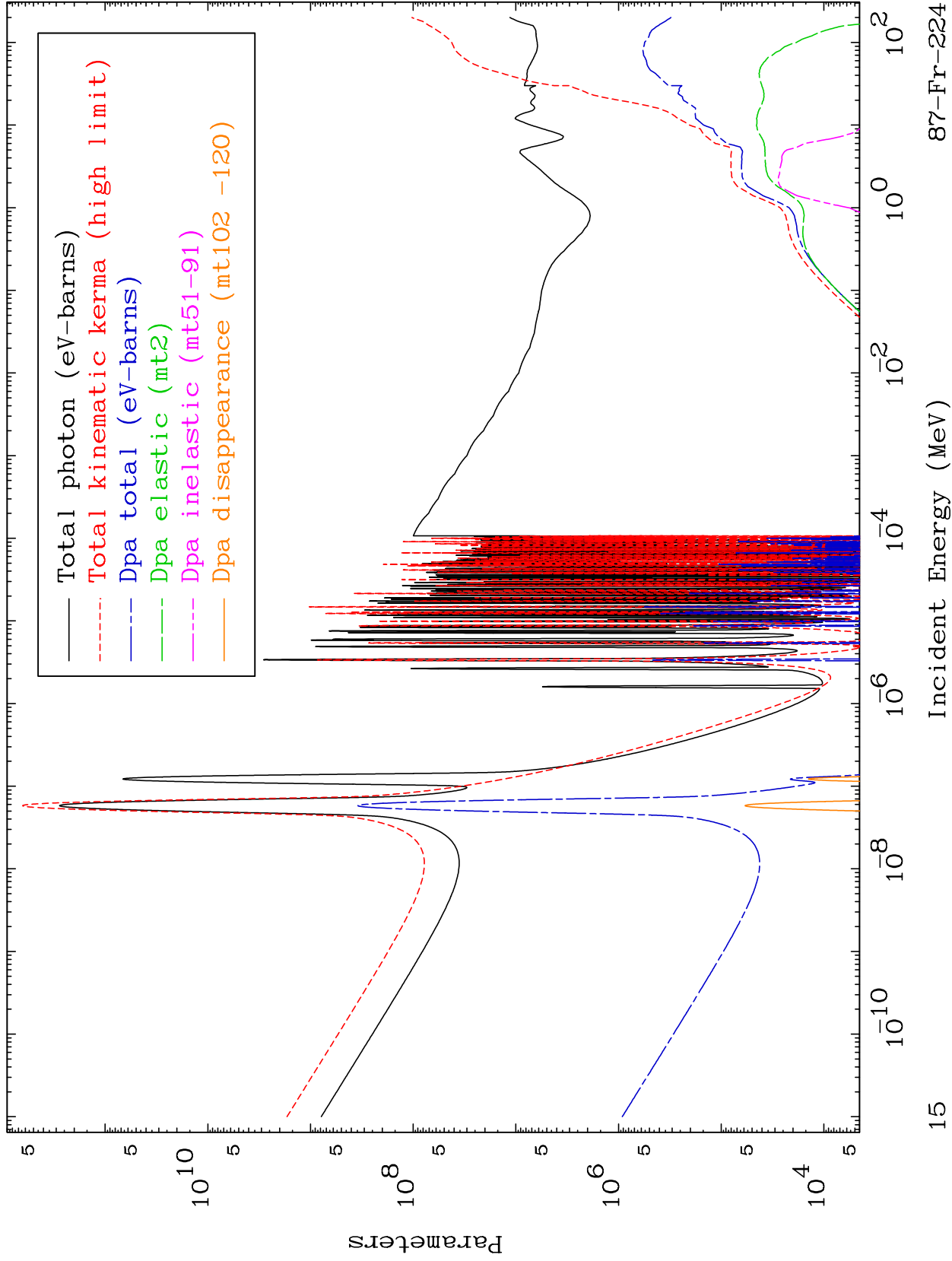


13

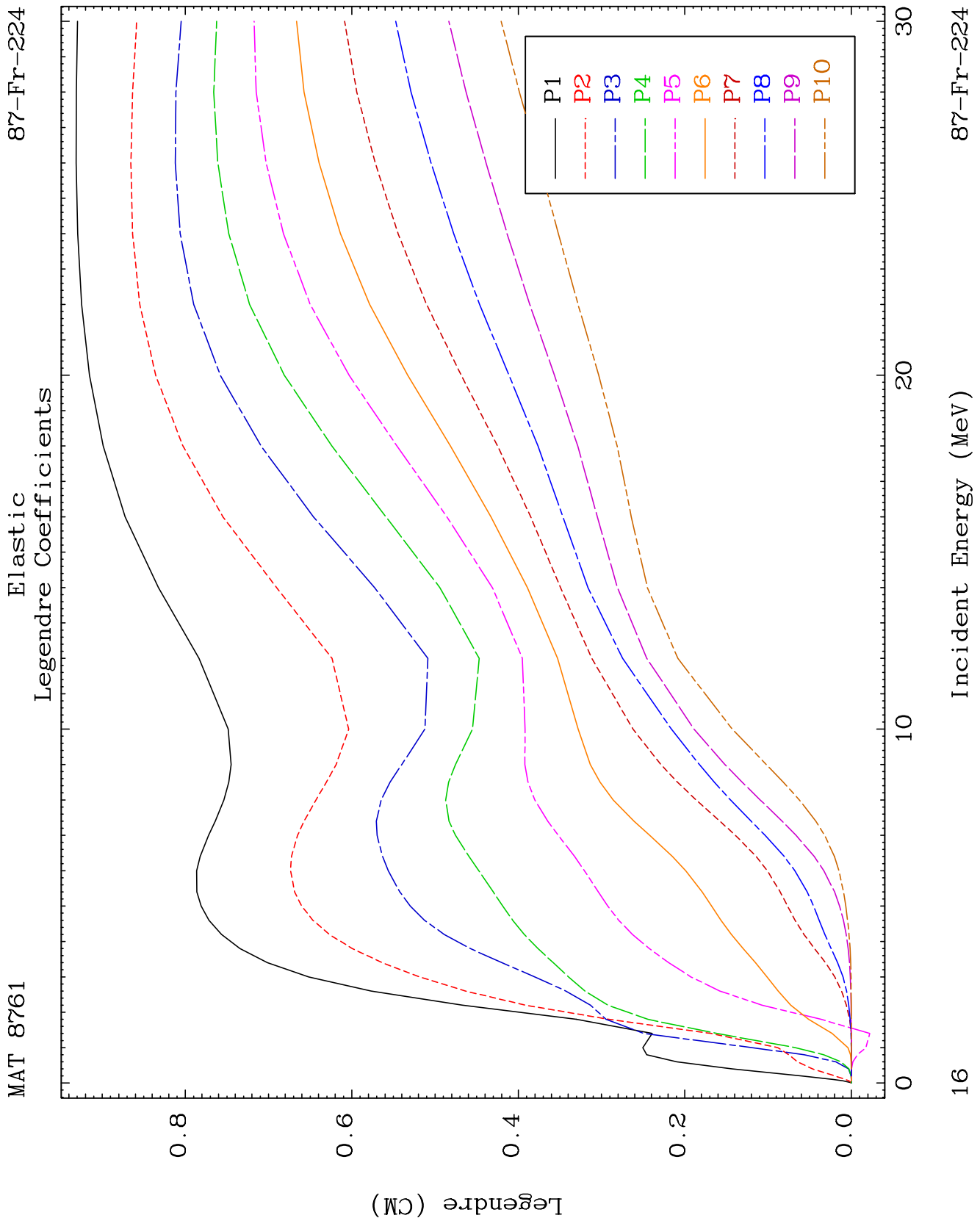
Incident Energy (MeV)

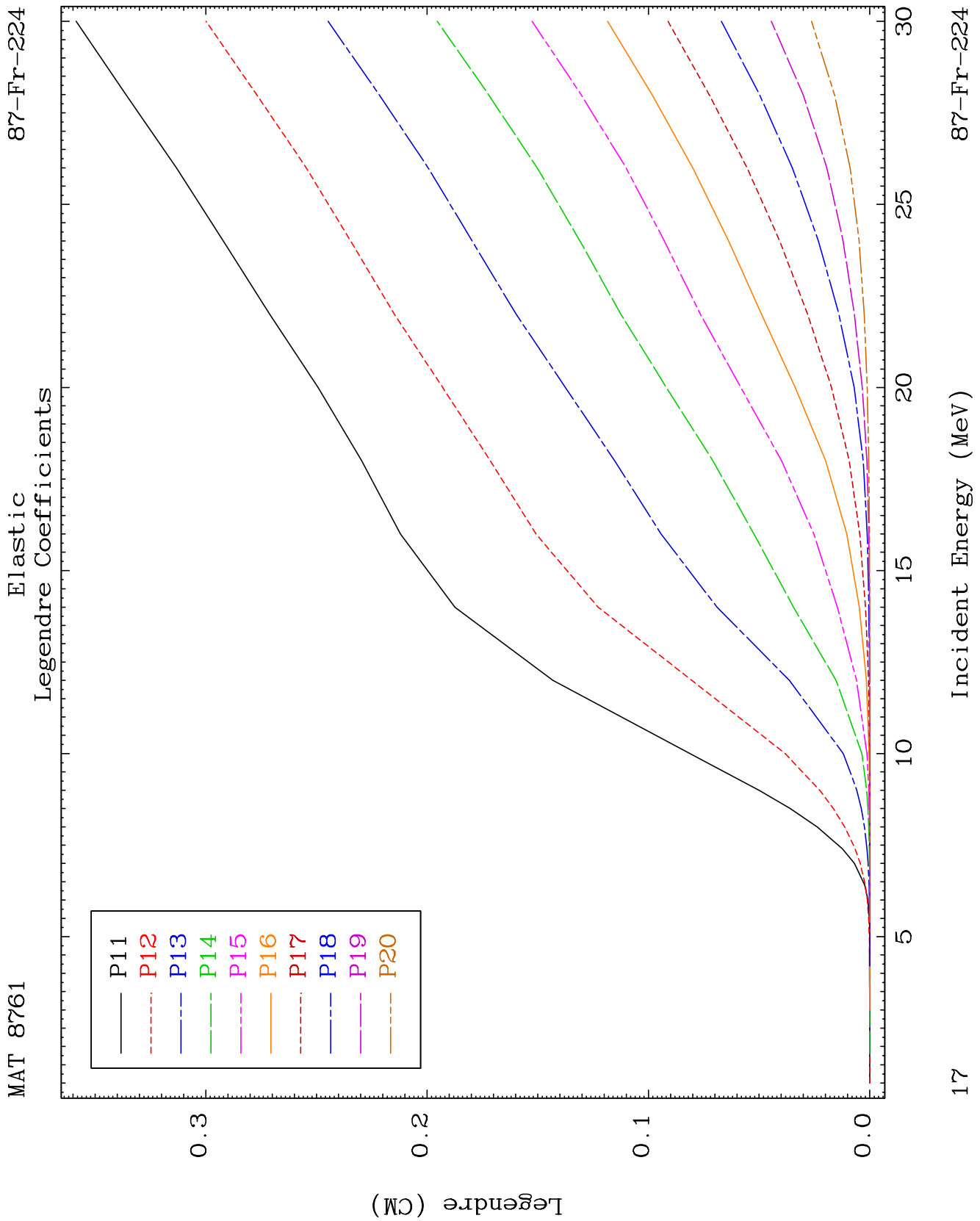
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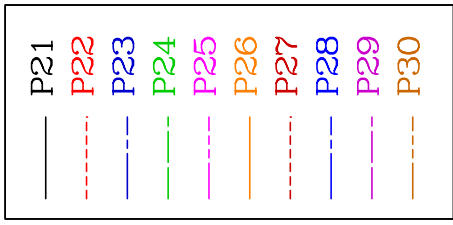
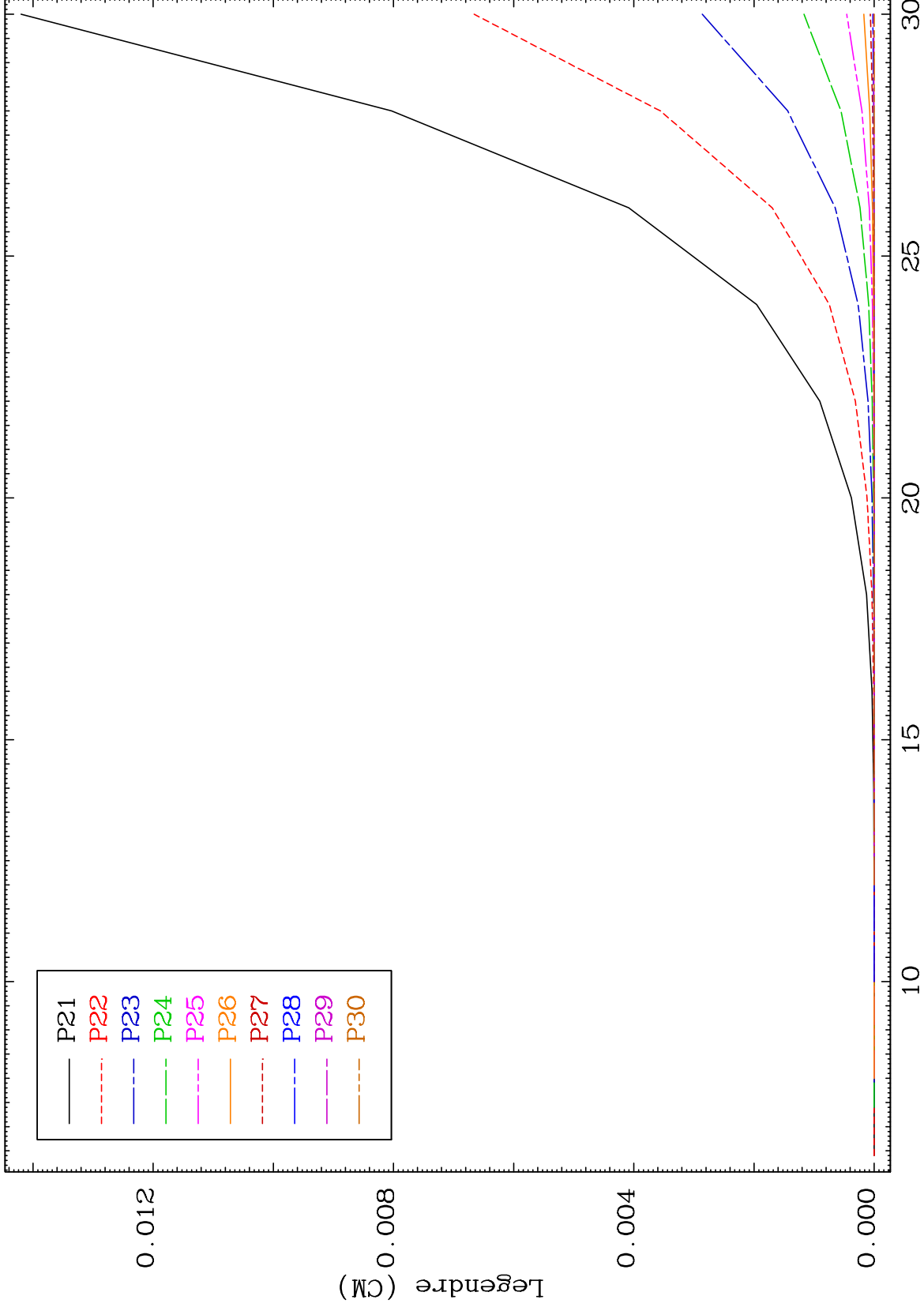




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Elastic  
Legendre Coefficients

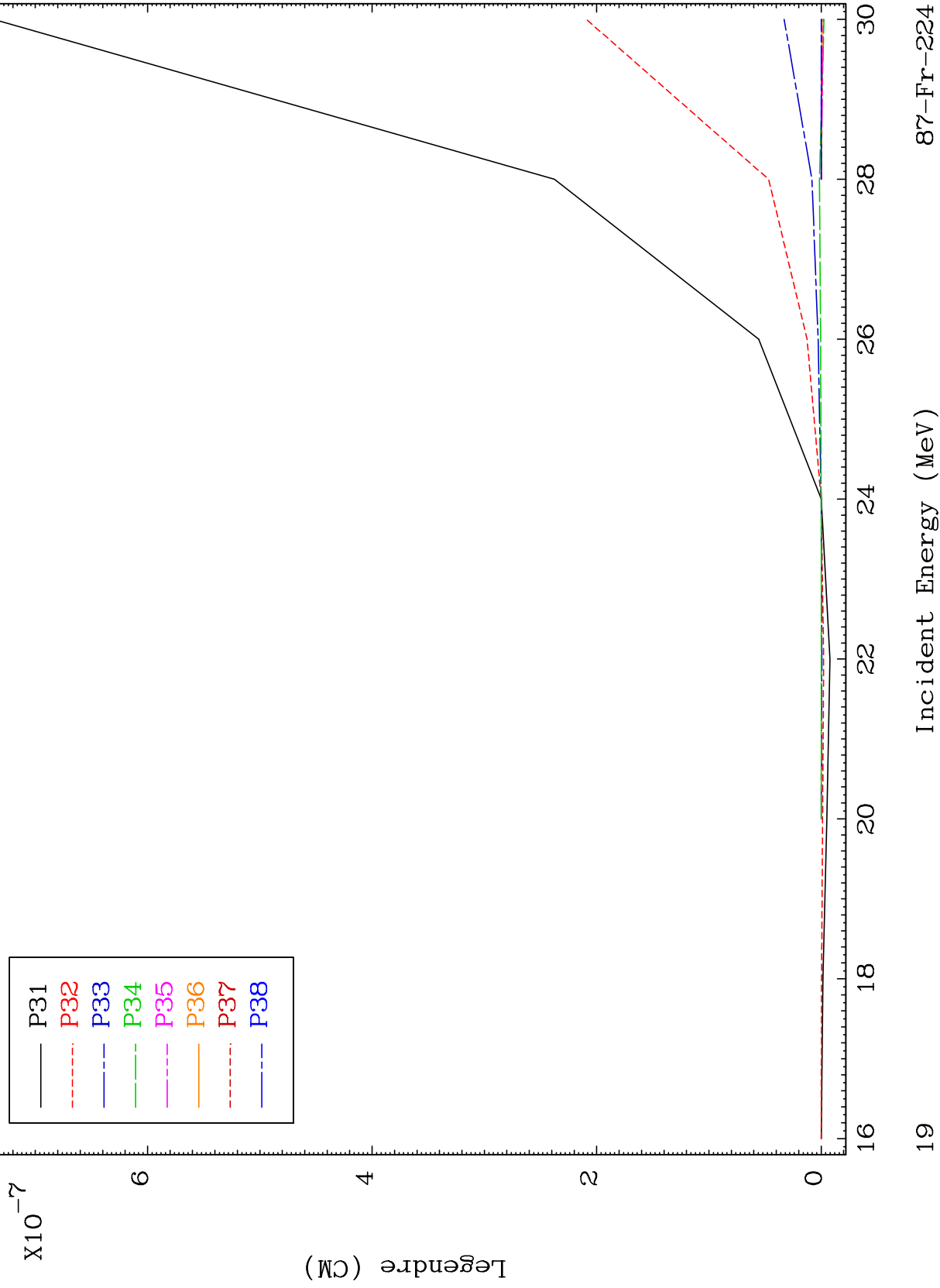
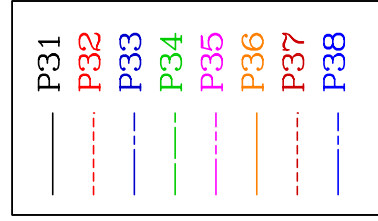
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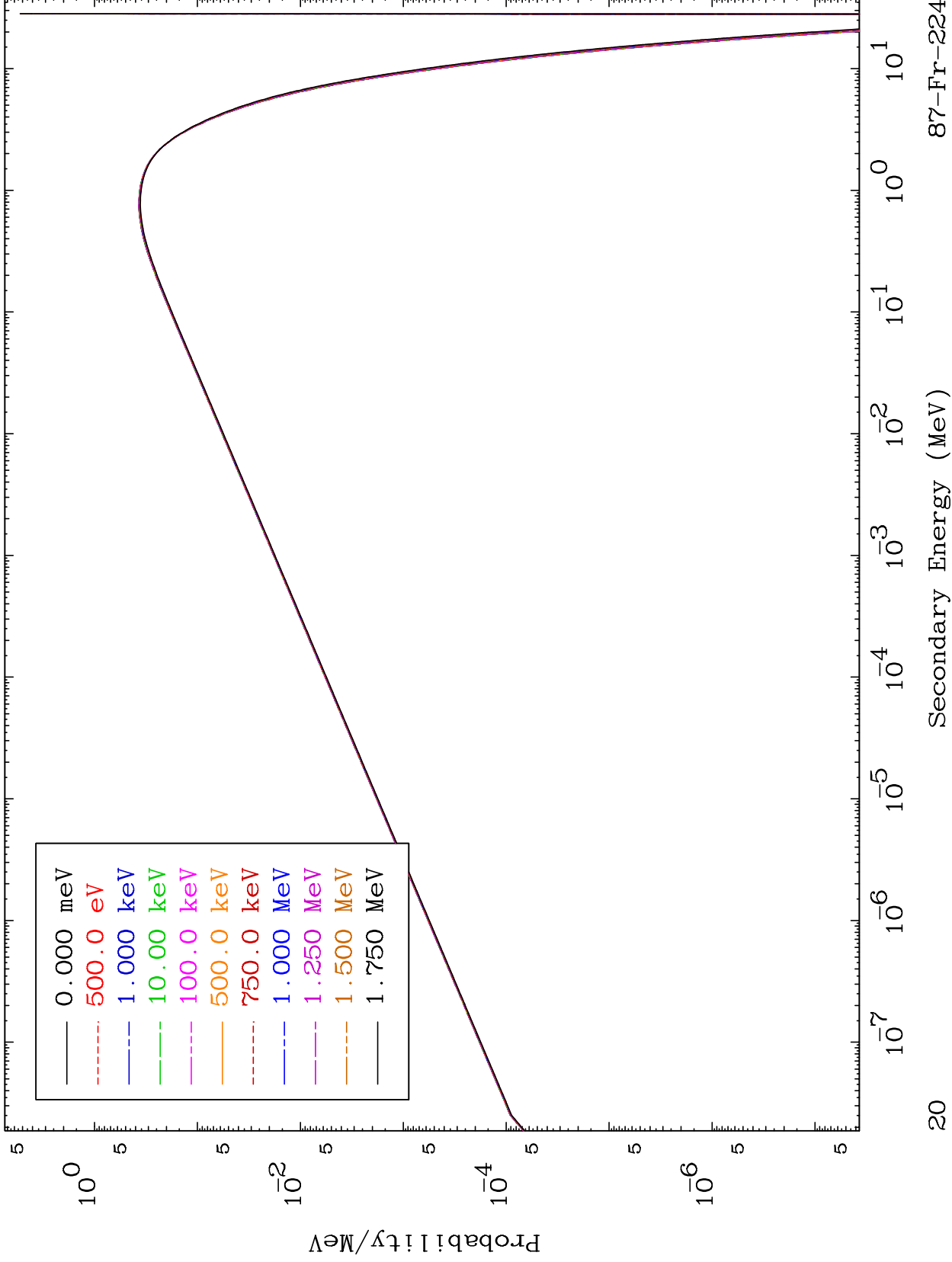
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Elastic  
Legendre Coefficients

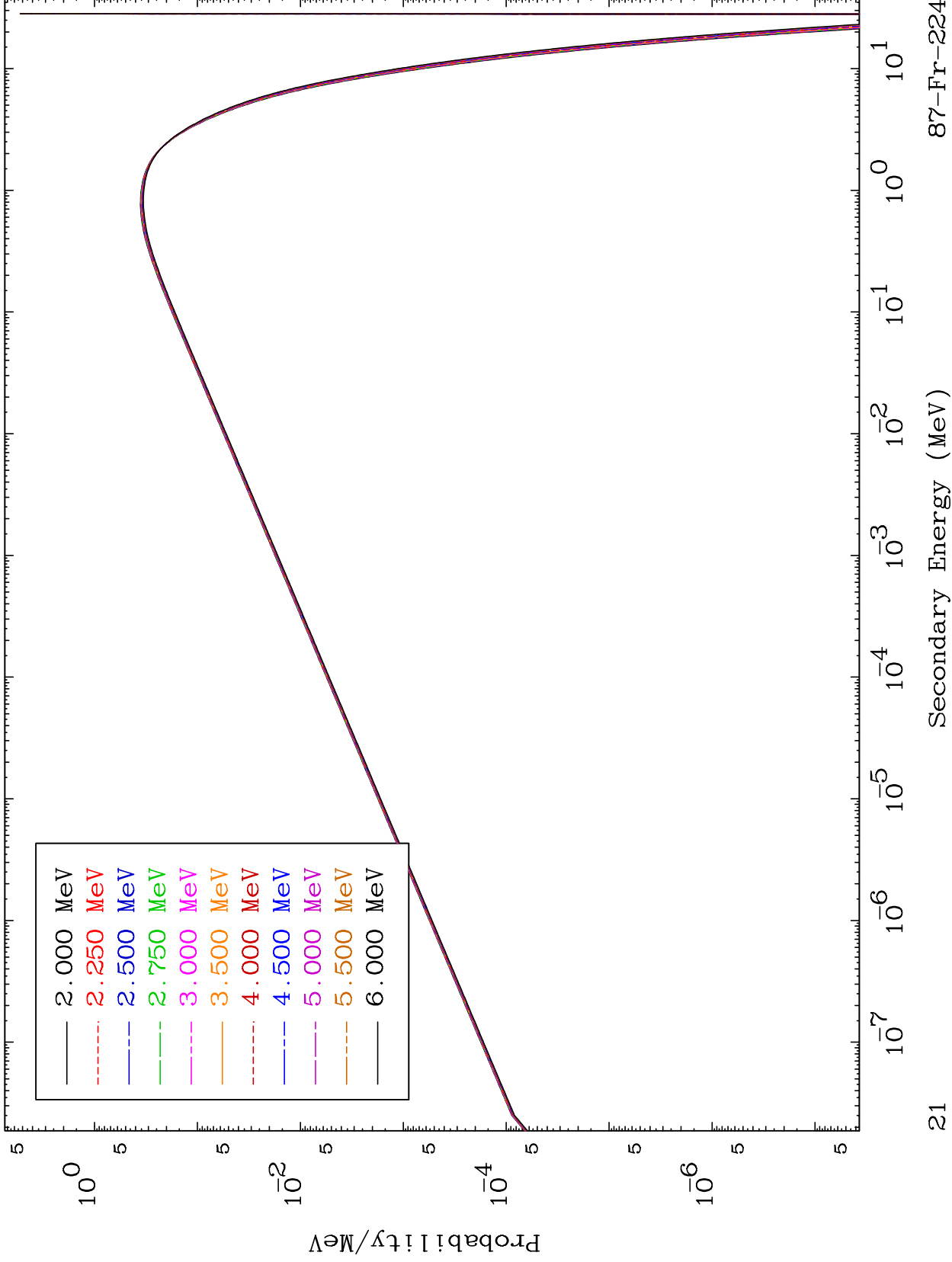
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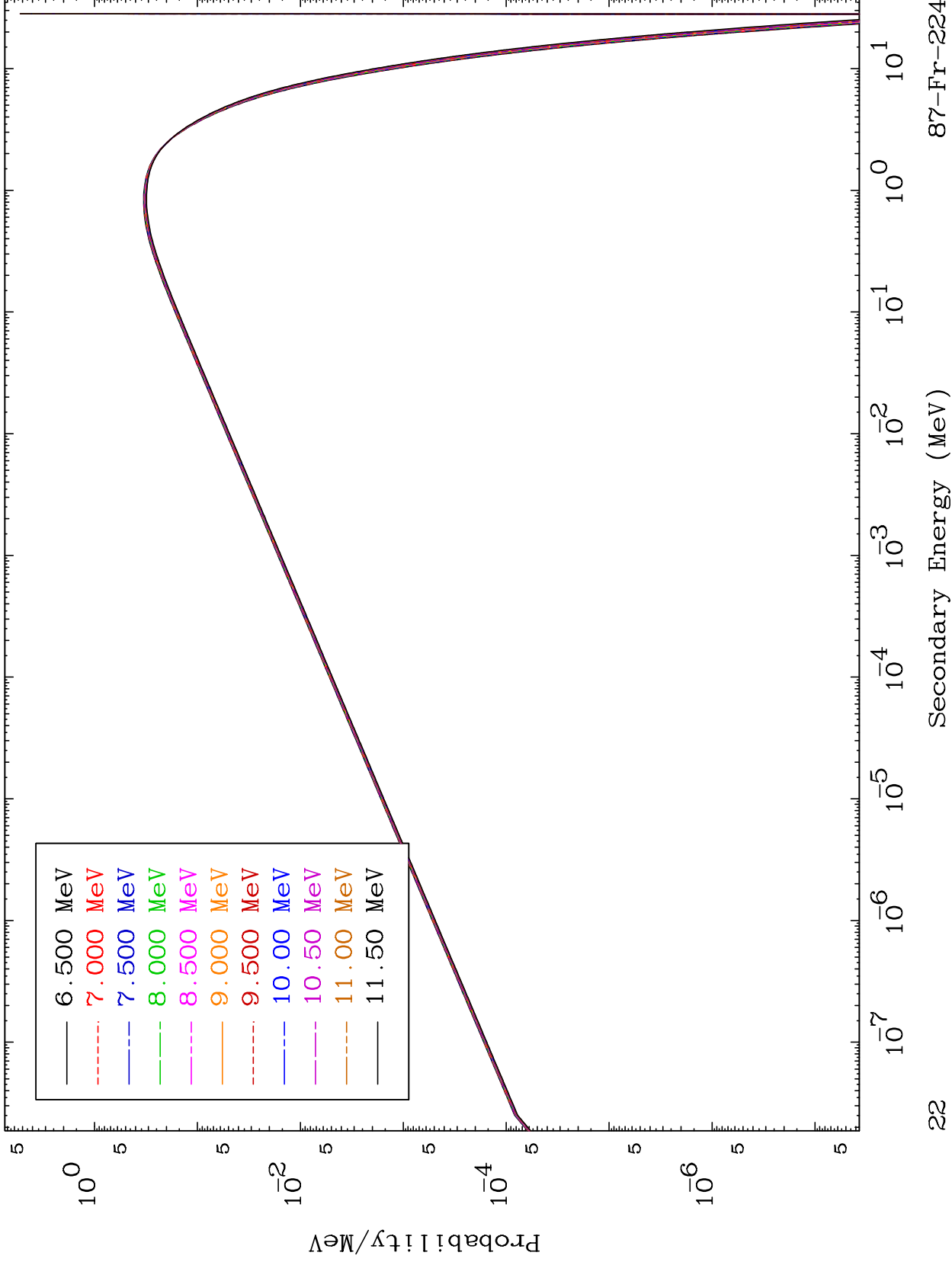
Fission  
Energy Distribution



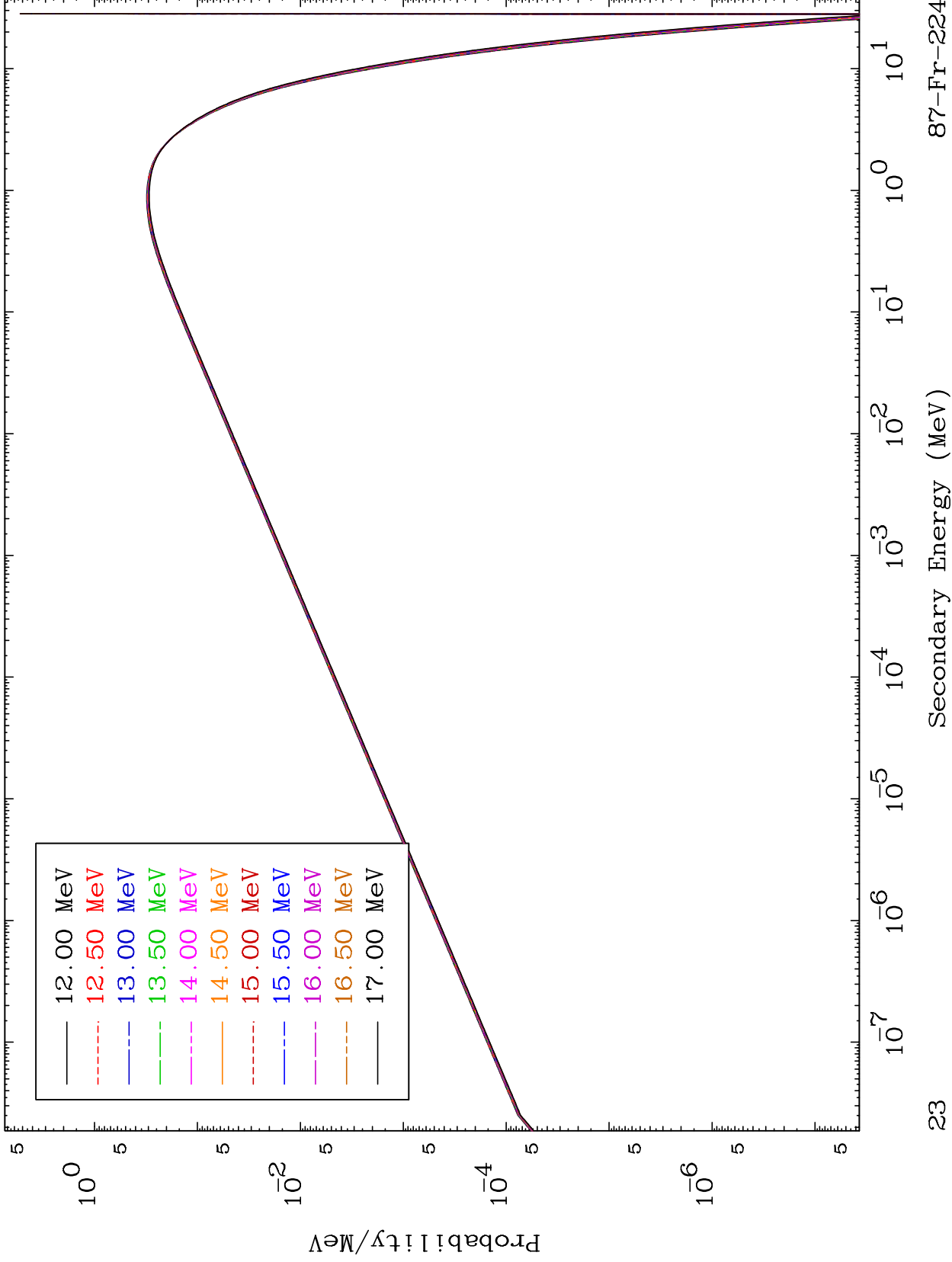
Fission  
Energy Distribution



Fission Energy Distribution

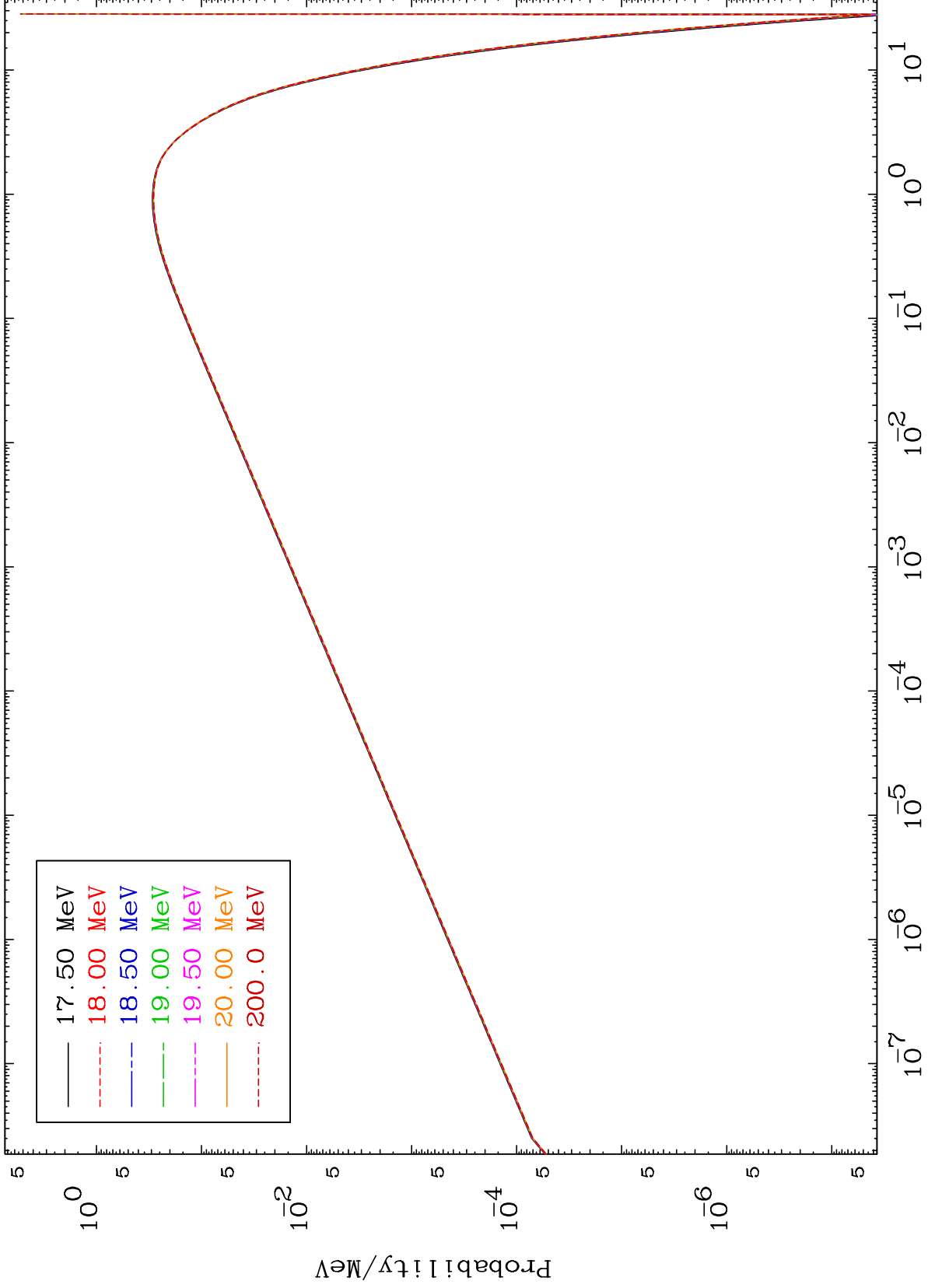


Fission  
Energy Distribution





Fission  
Energy Distribution

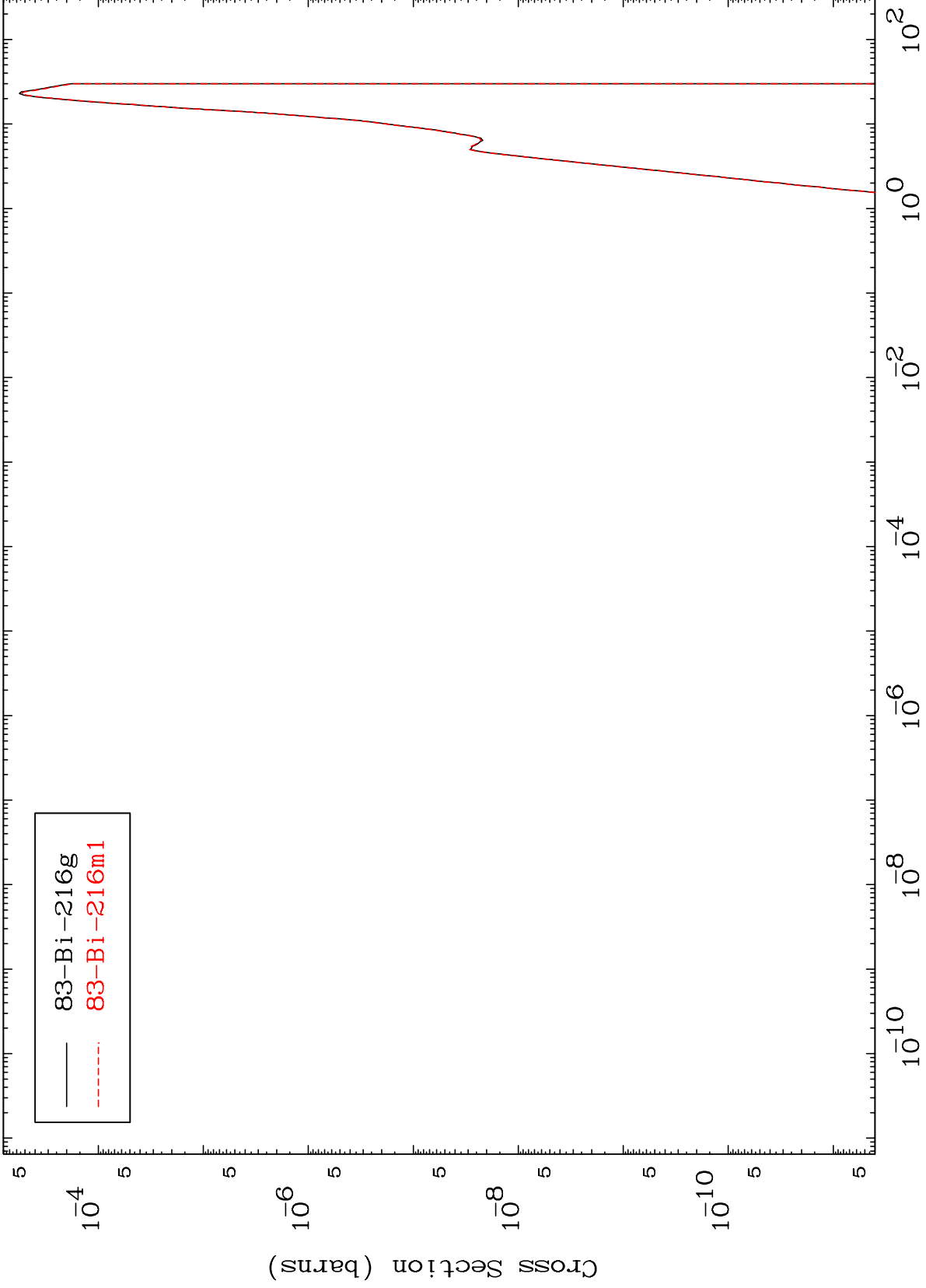


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(n,n') 2α

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Radionuclide Production Cross Section



25

Incident Energy (MeV)

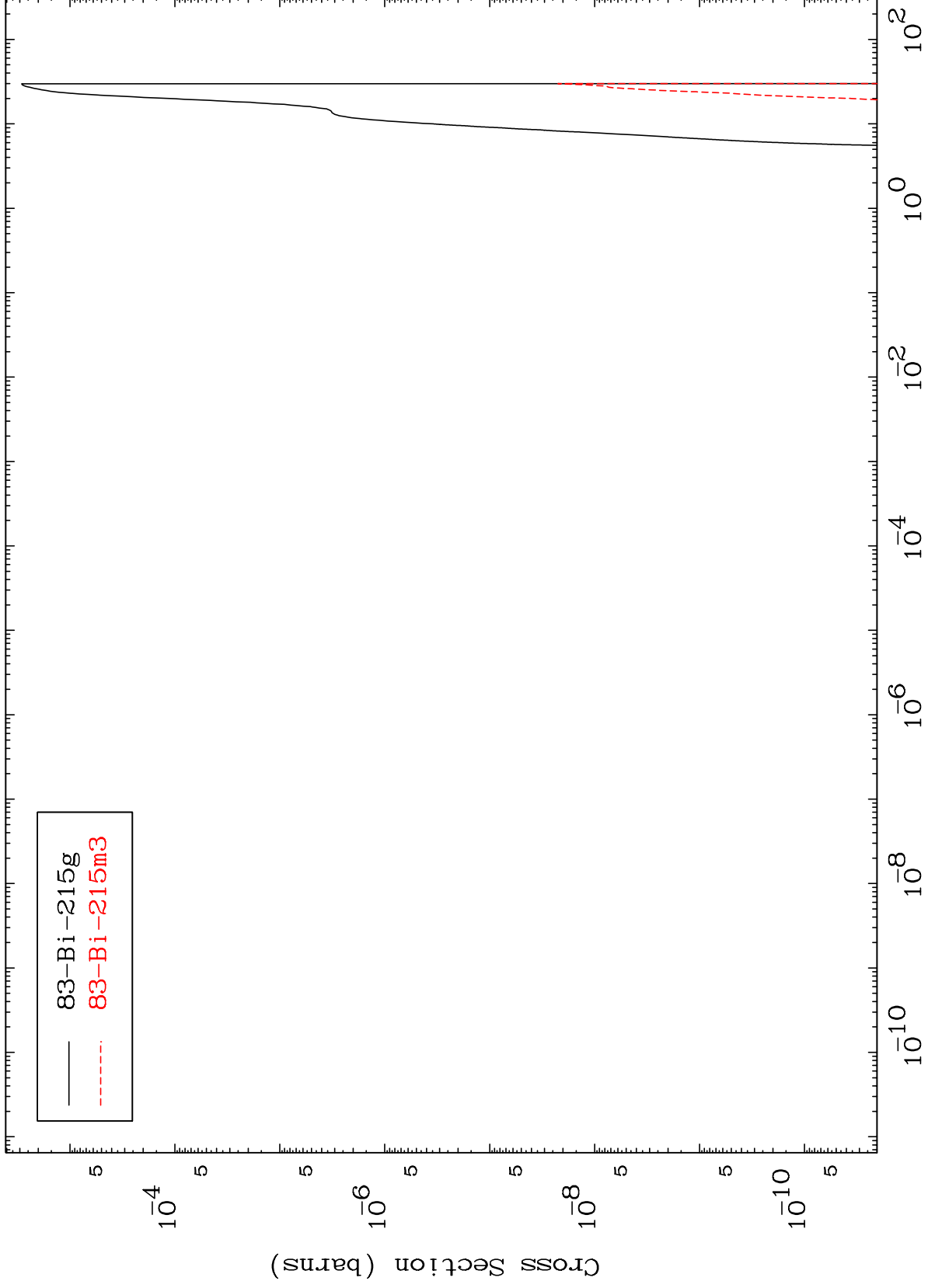
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(n,2n) 2α

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Radionuclide Production Cross Section



26

Incident Energy (MeV)

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