

Program EVALPLOT  
(Version 2015-2)

by

Dermott E. Cullen  
(Present Contact Information)

Dermott E. Cullen  
1466 Hudson Way  
Livermore, CA 94550  
U.S.A.

Tele: 925-443-1911

E.Mail:redcullen1@comcast.net  
Web:home.comcast.net/~redcullen1

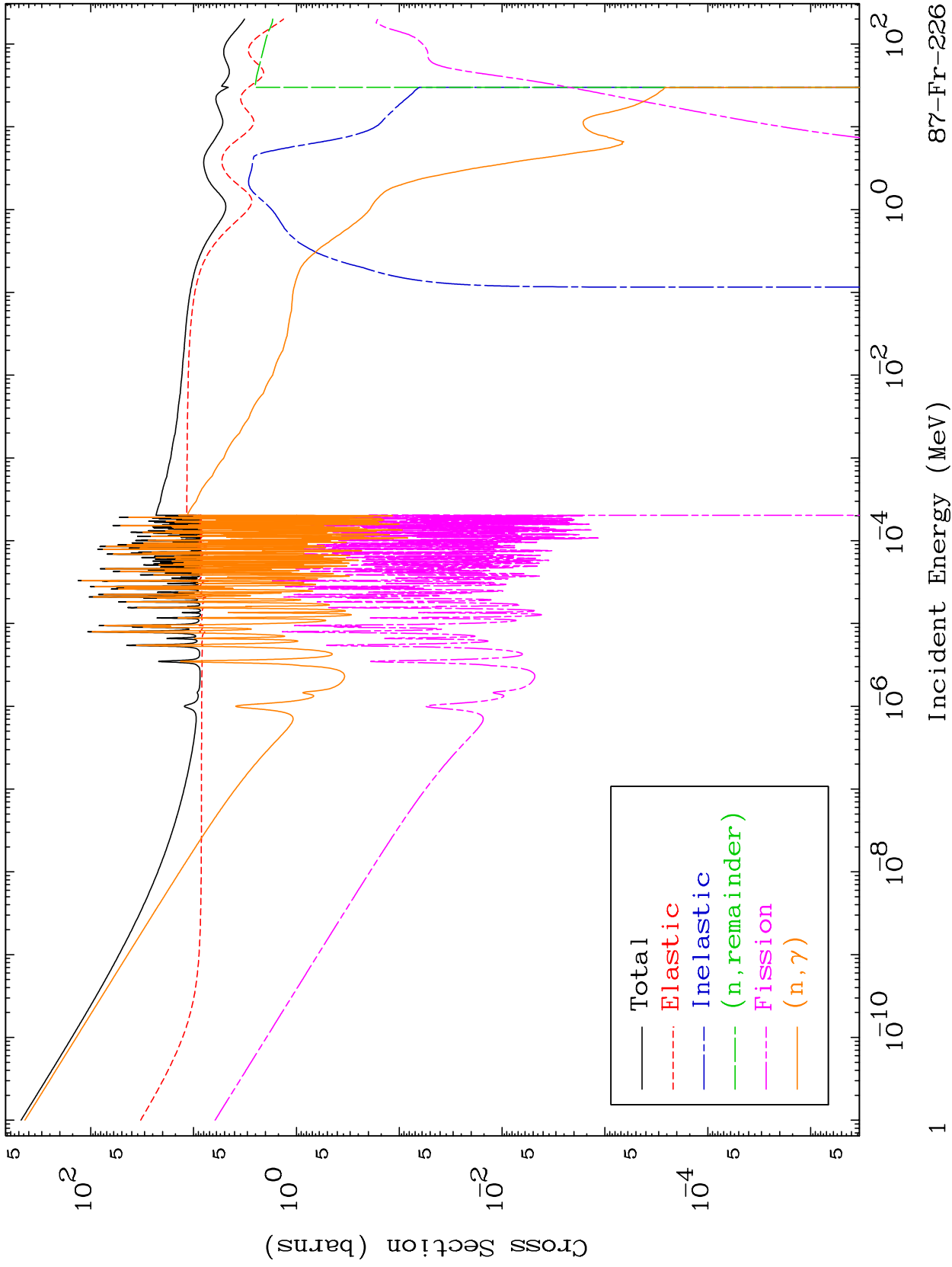
Press Mouse Button to Start

MAT 8767

Major

293 Kelvin Cross Sections

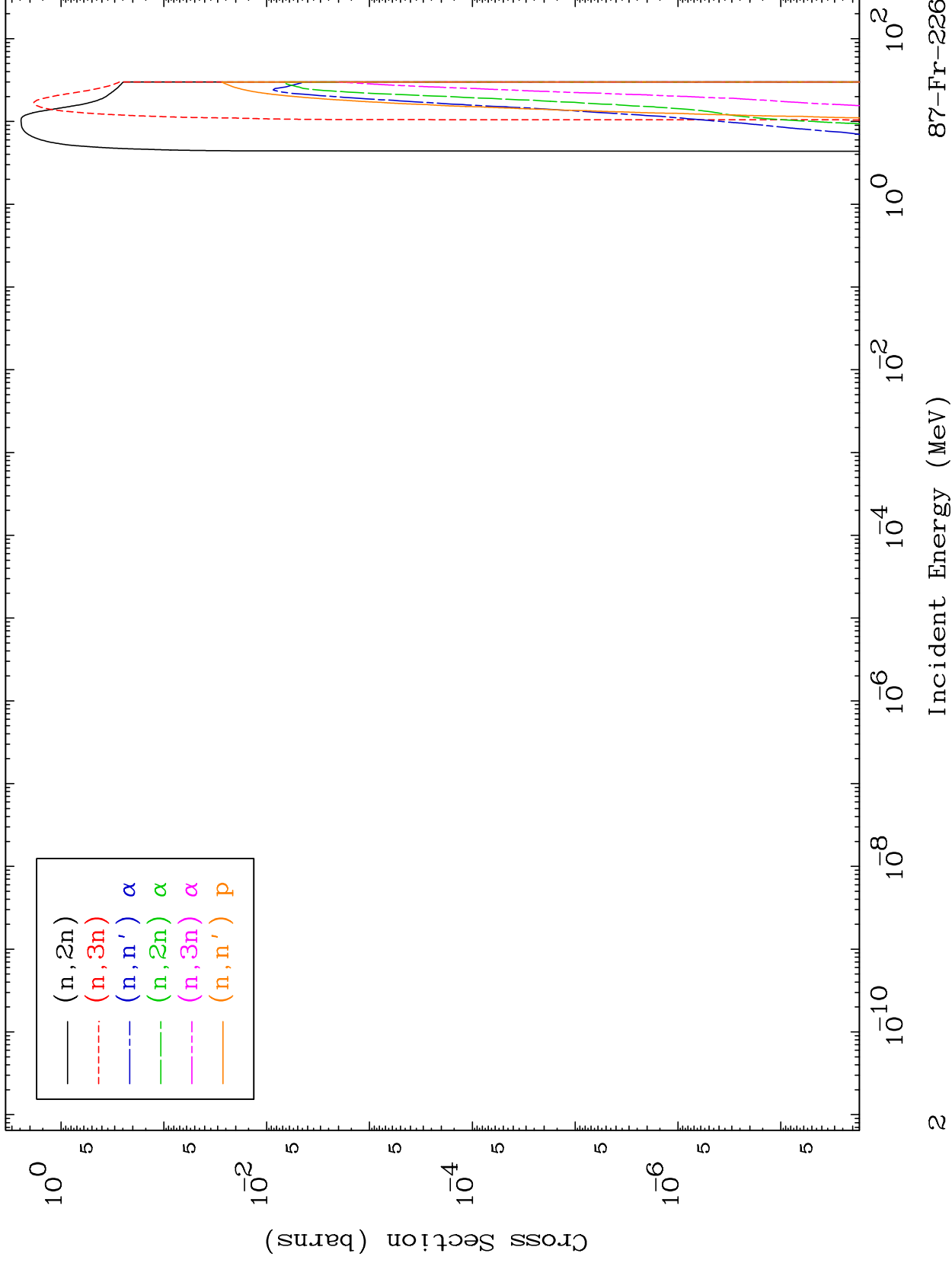
87-Fr-226



MAT 8767

Neutron Production  
293 Kelvin Cross Sections

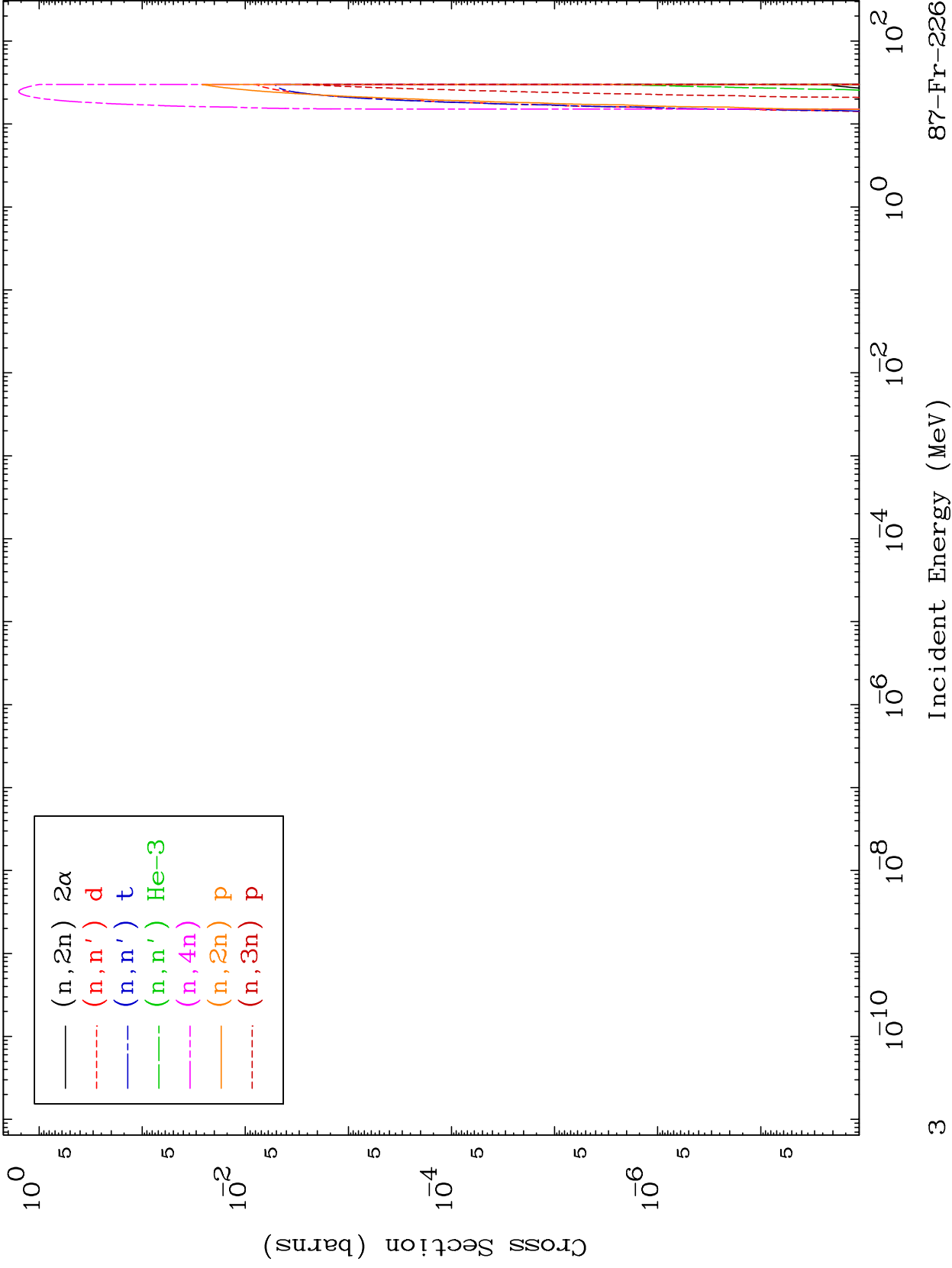
87-Fr-226



MAT 8767

Neutron Production  
293 Kelvin Cross Sections

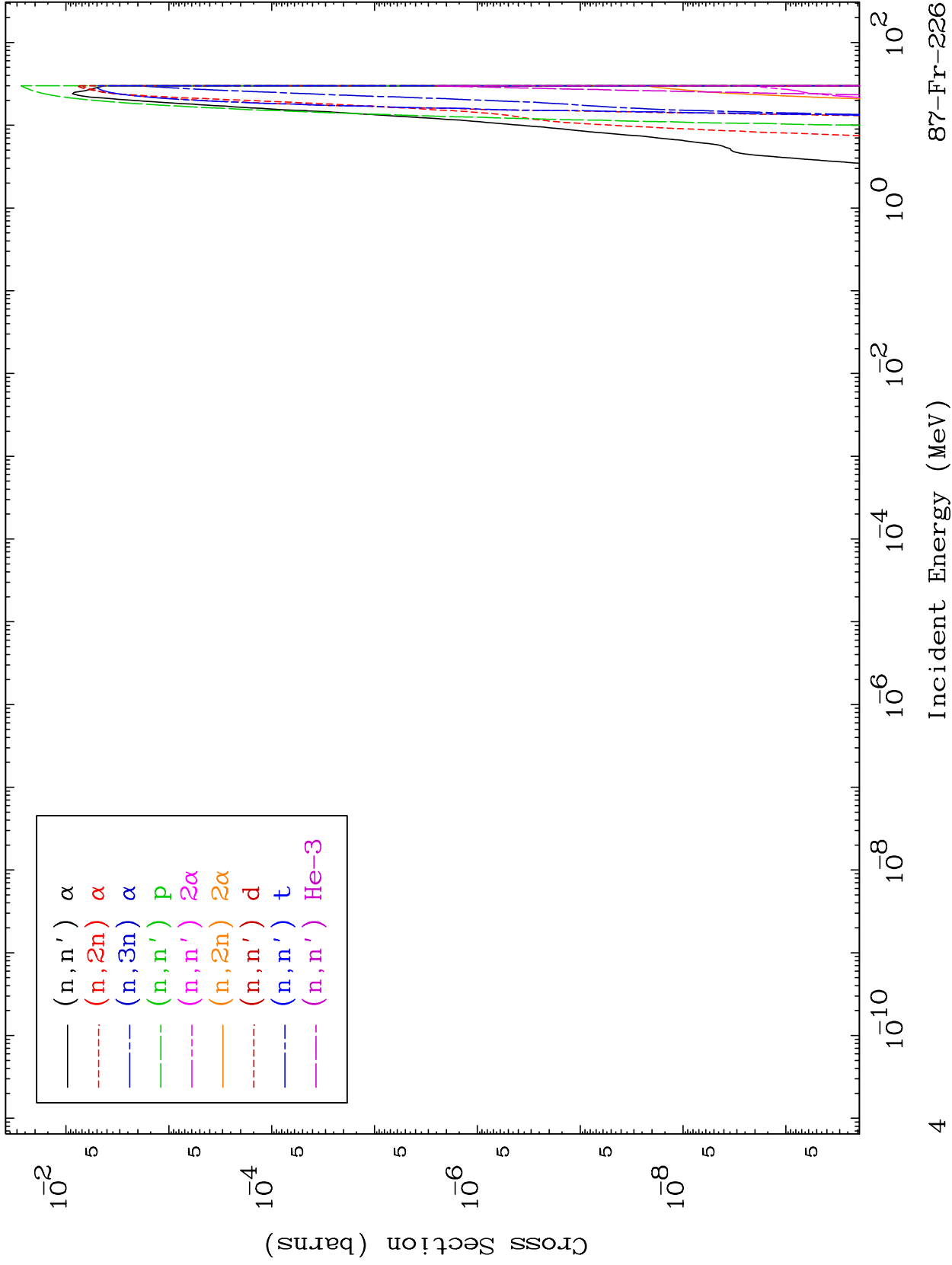
87-Fr-226

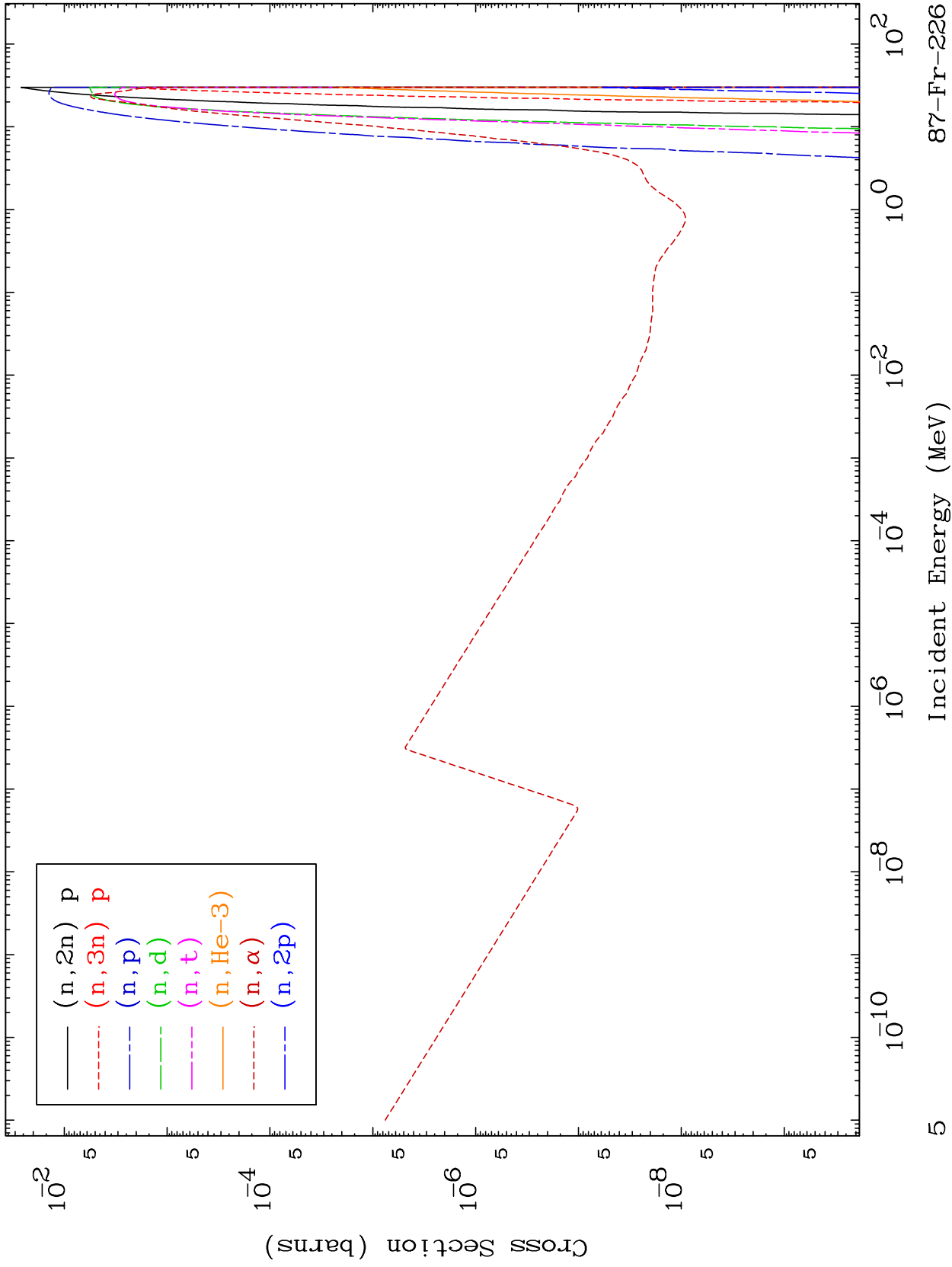


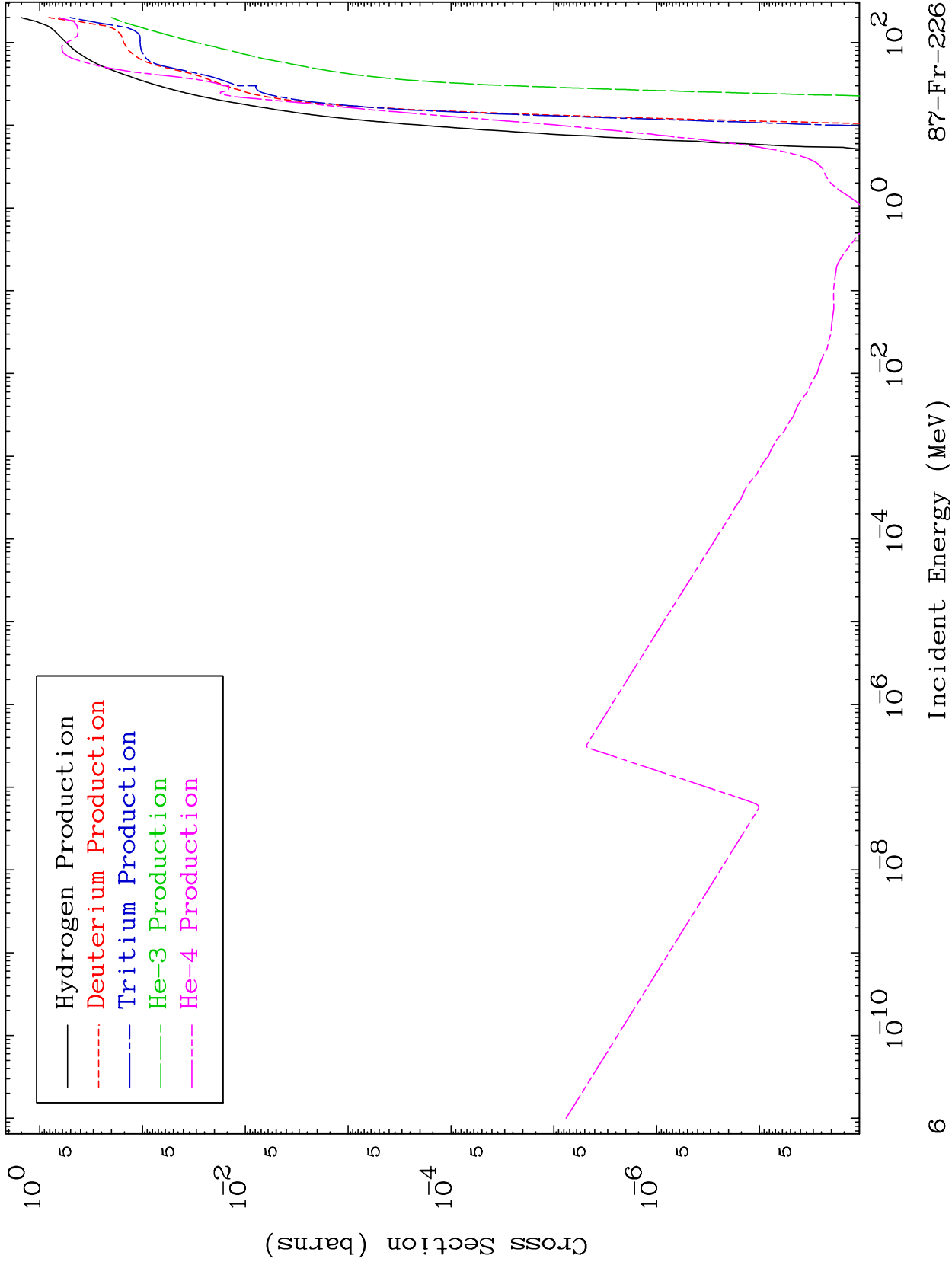
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Charged Particle  
293 Kelvin Cross Sections

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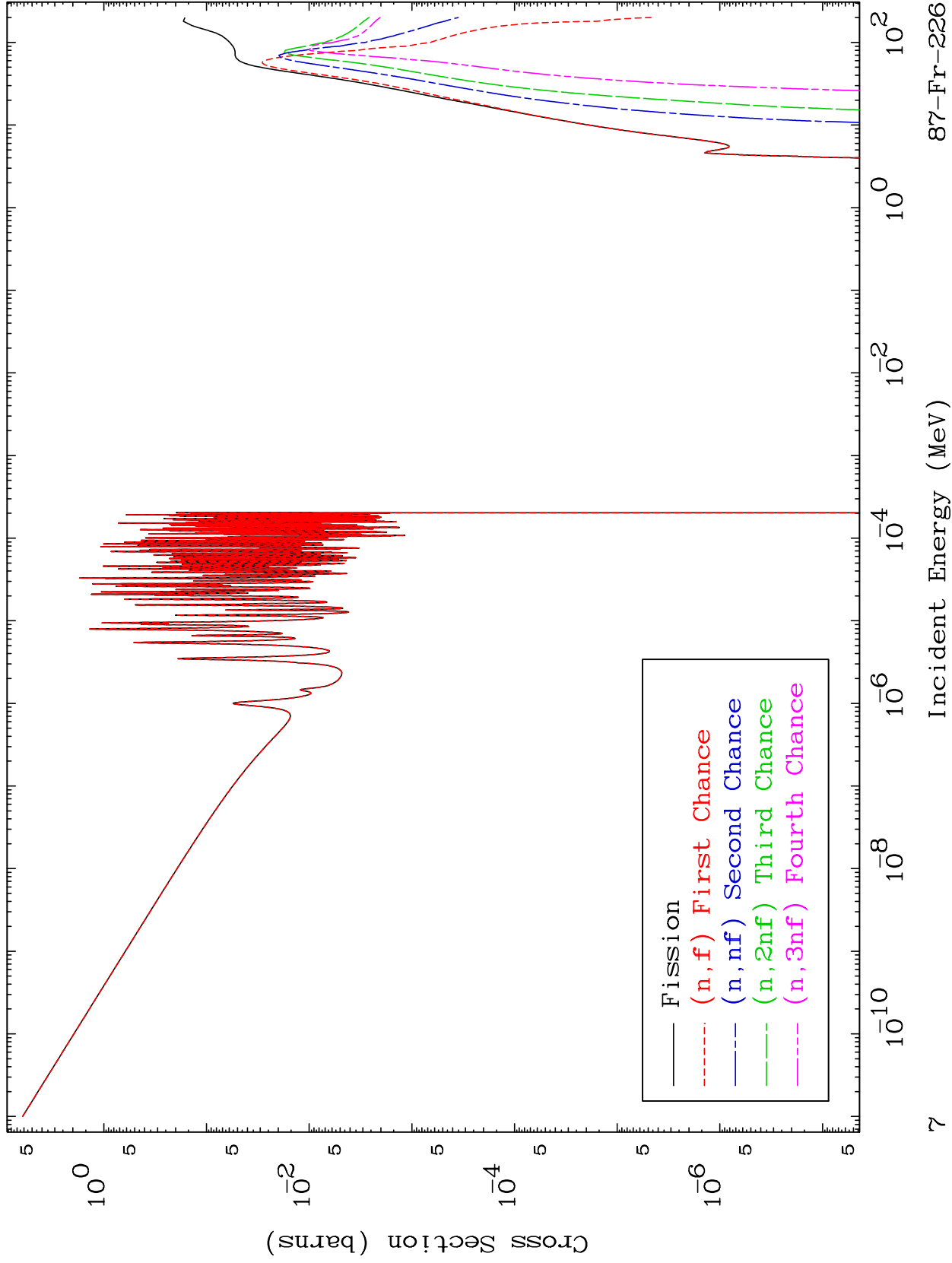




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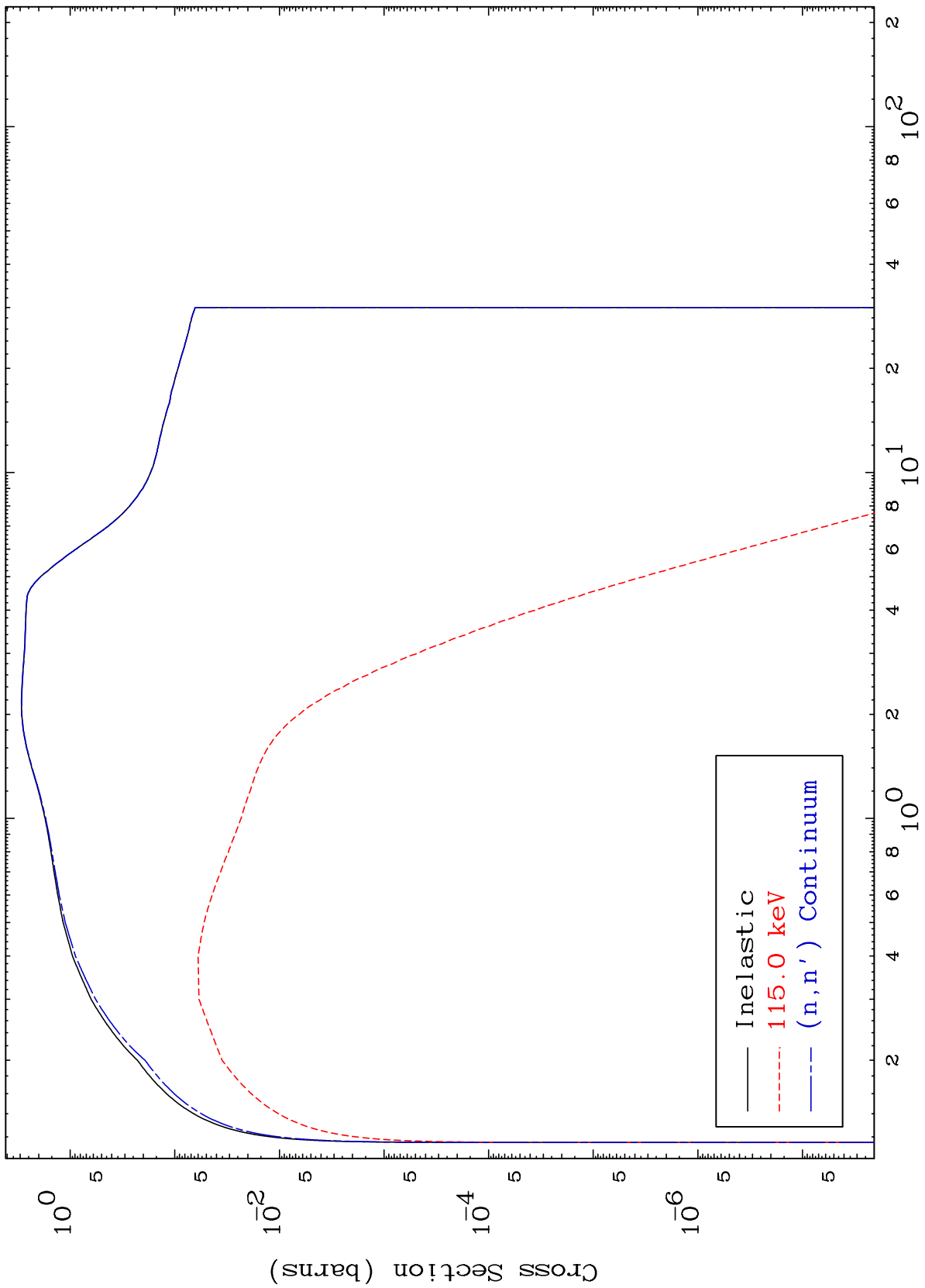
Fission  
293 Kelvin Cross Sections

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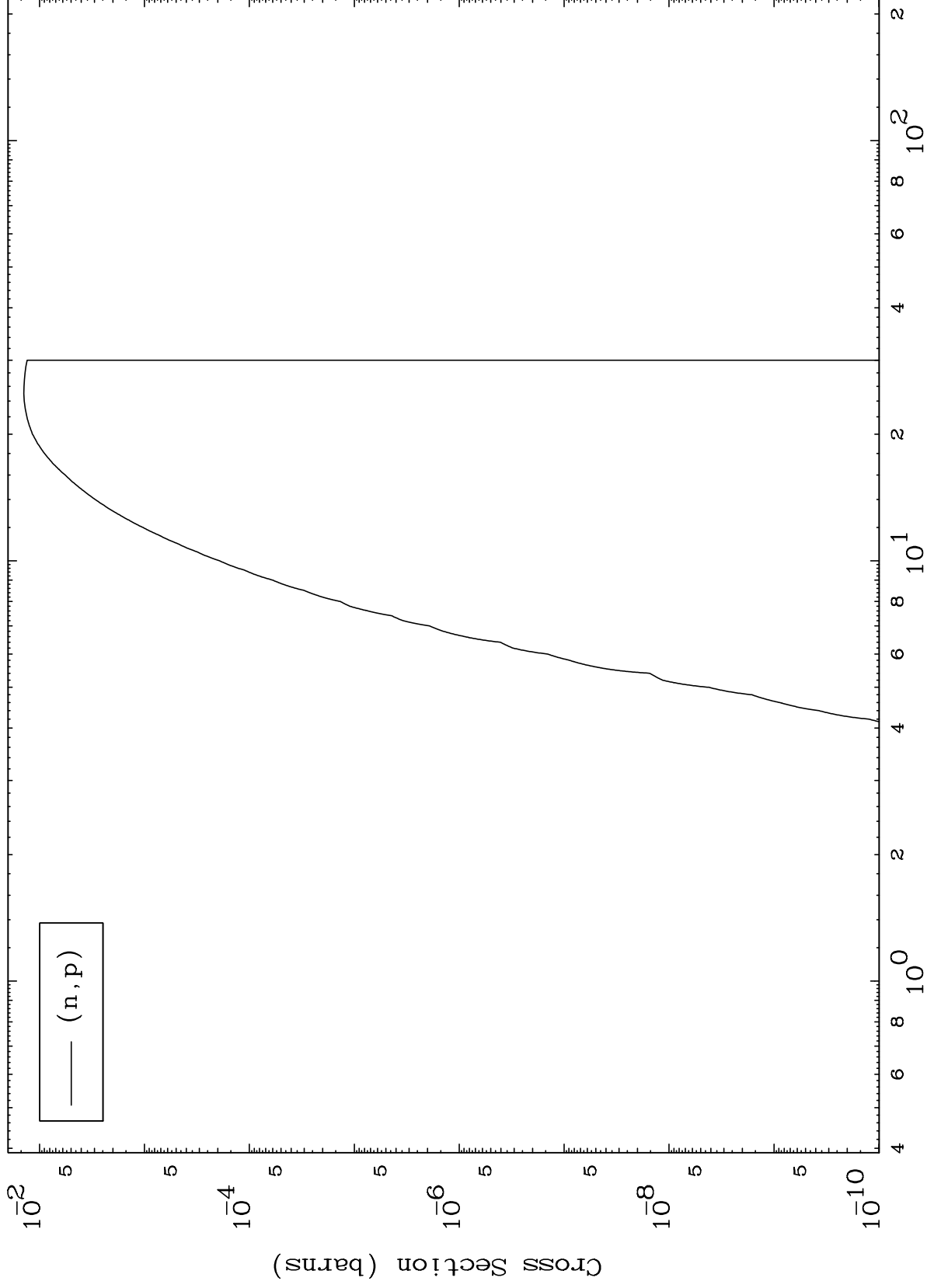
(n,n') Level  
293 Kelvin Cross Sections



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(n,p) Levels  
293 Kelvin Cross Sections

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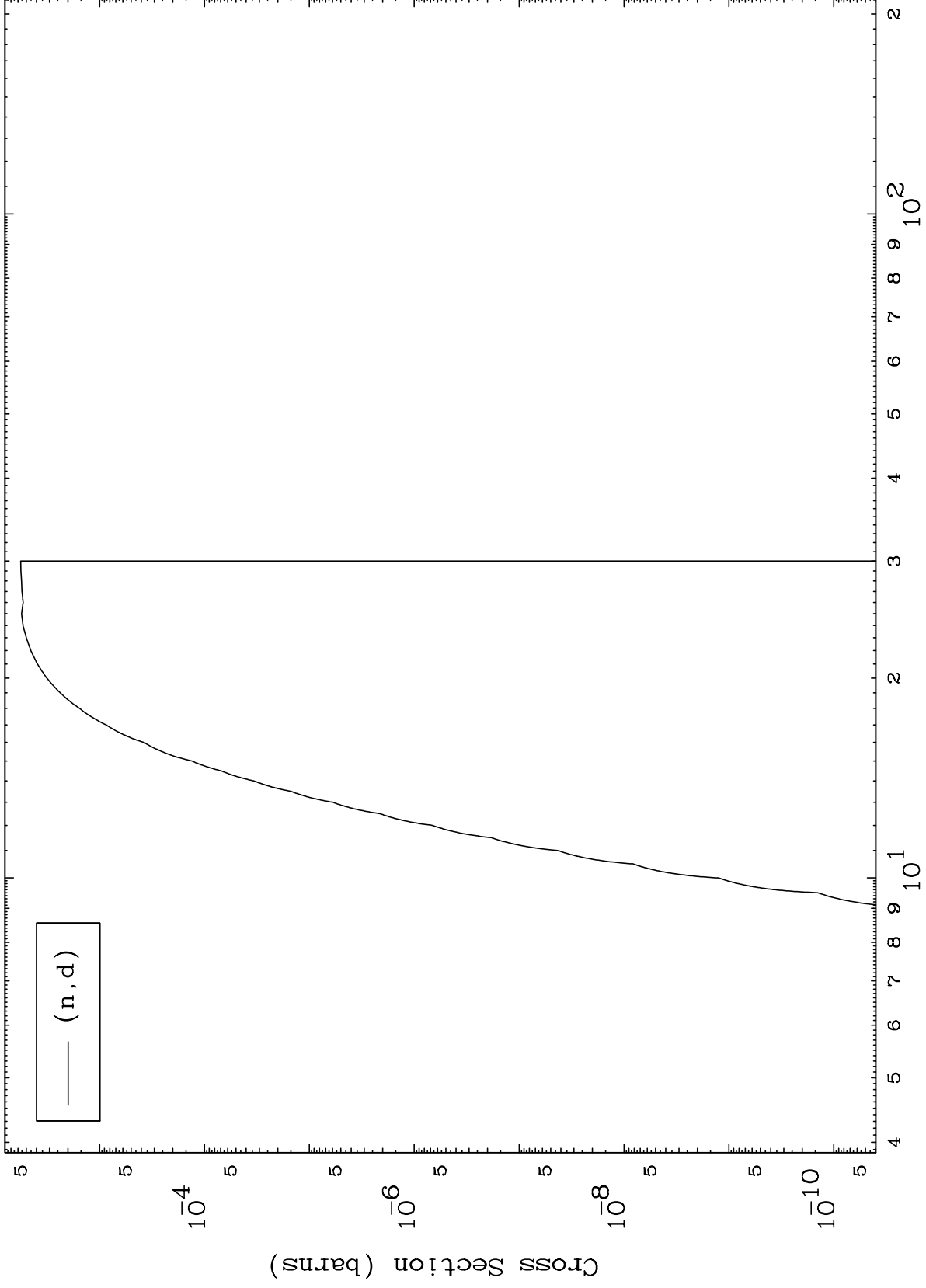
Incident Energy (MeV)

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(n,d) Levels  
293 Kelvin Cross Sections

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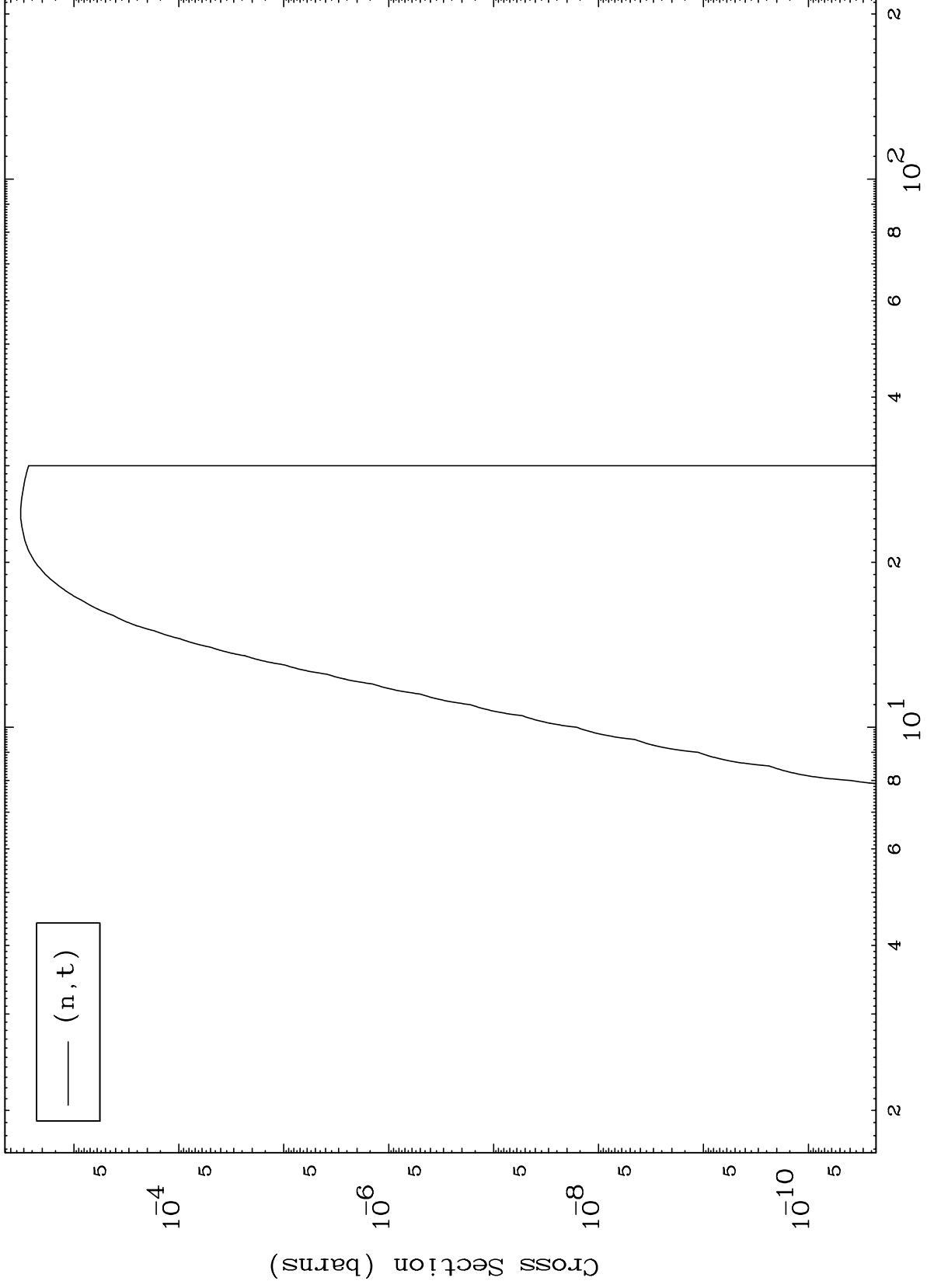
Incident Energy (MeV)

87-Fr-226

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(n,t) Levels  
293 Kelvin Cross Sections

87-Fr-226



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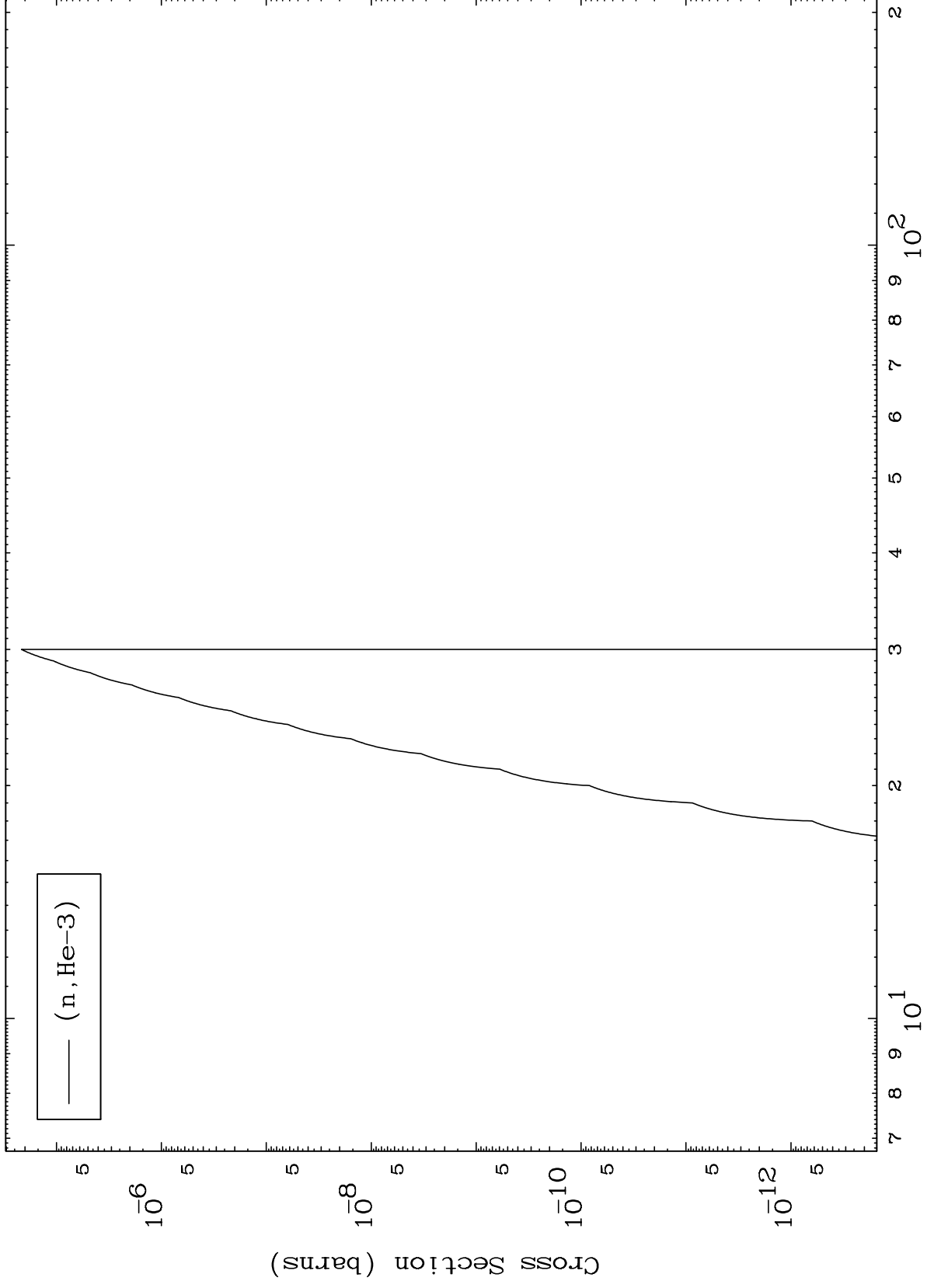
Incident Energy (MeV)

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(n,He3) Levels  
293 Kelvin Cross Sections

87-Fr-226



12

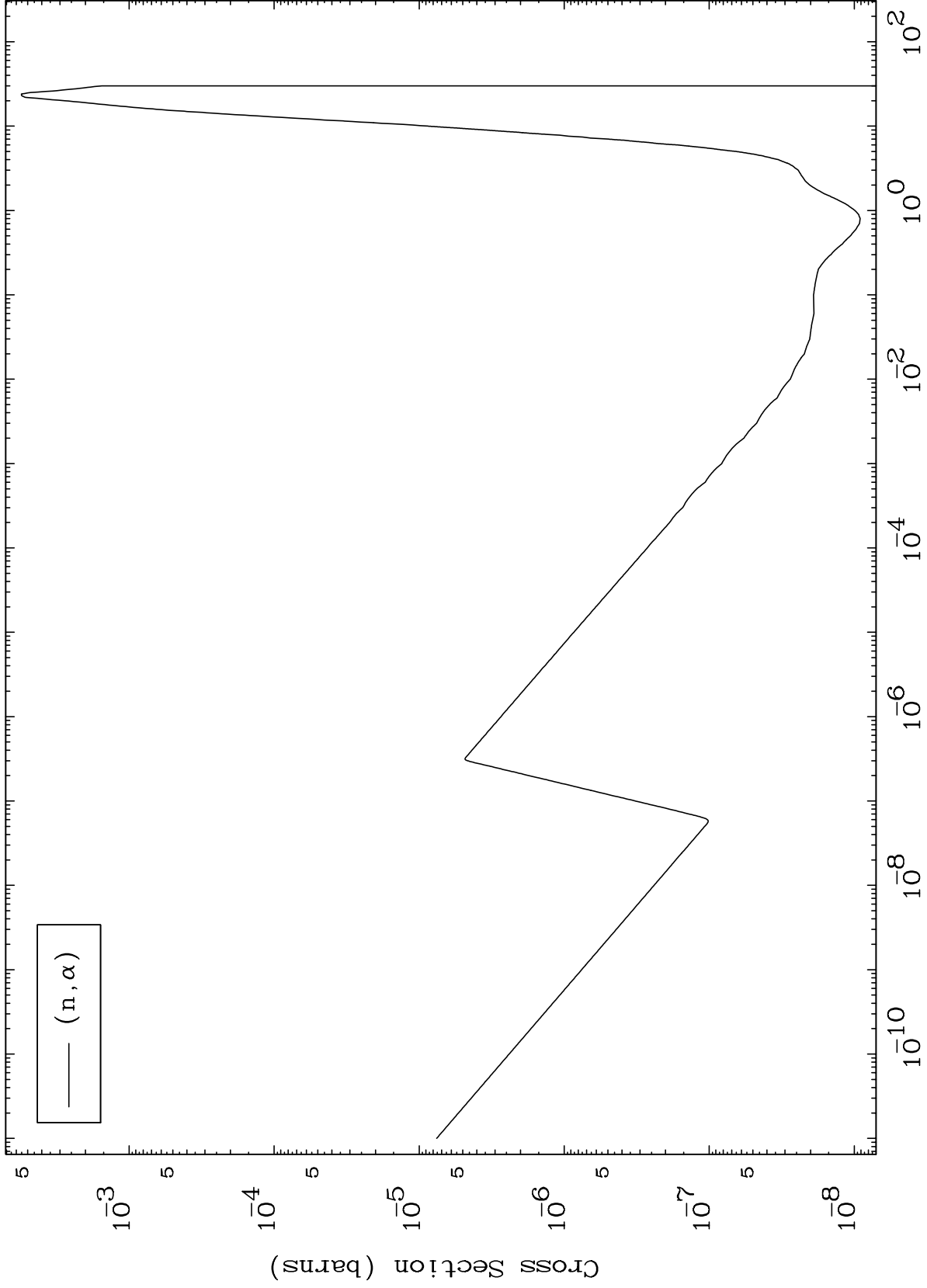
Incident Energy (MeV)

87-Fr-226

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(n,  $\alpha$ ) Levels  
293 Kelvin Cross Sections

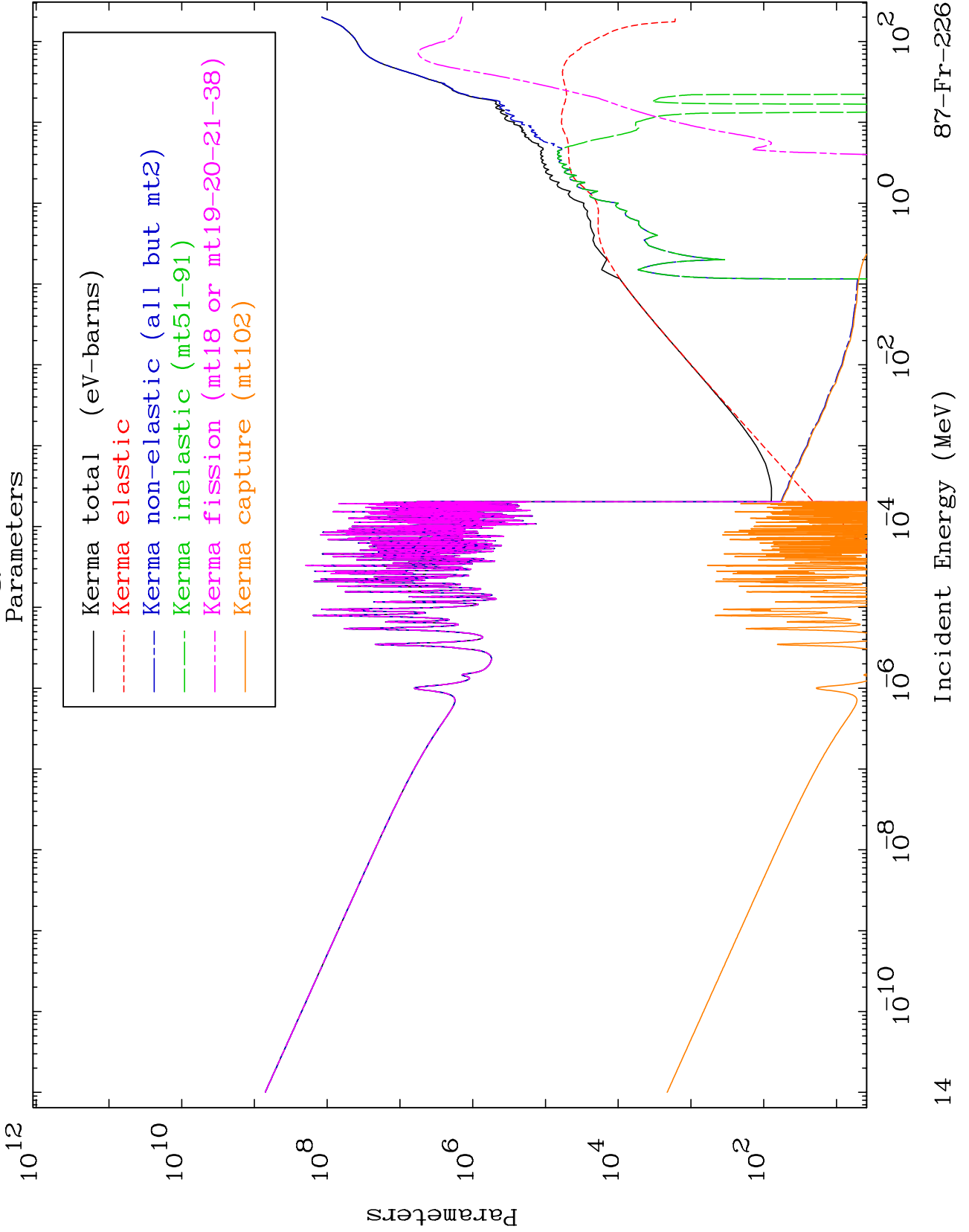
87-Fr-226

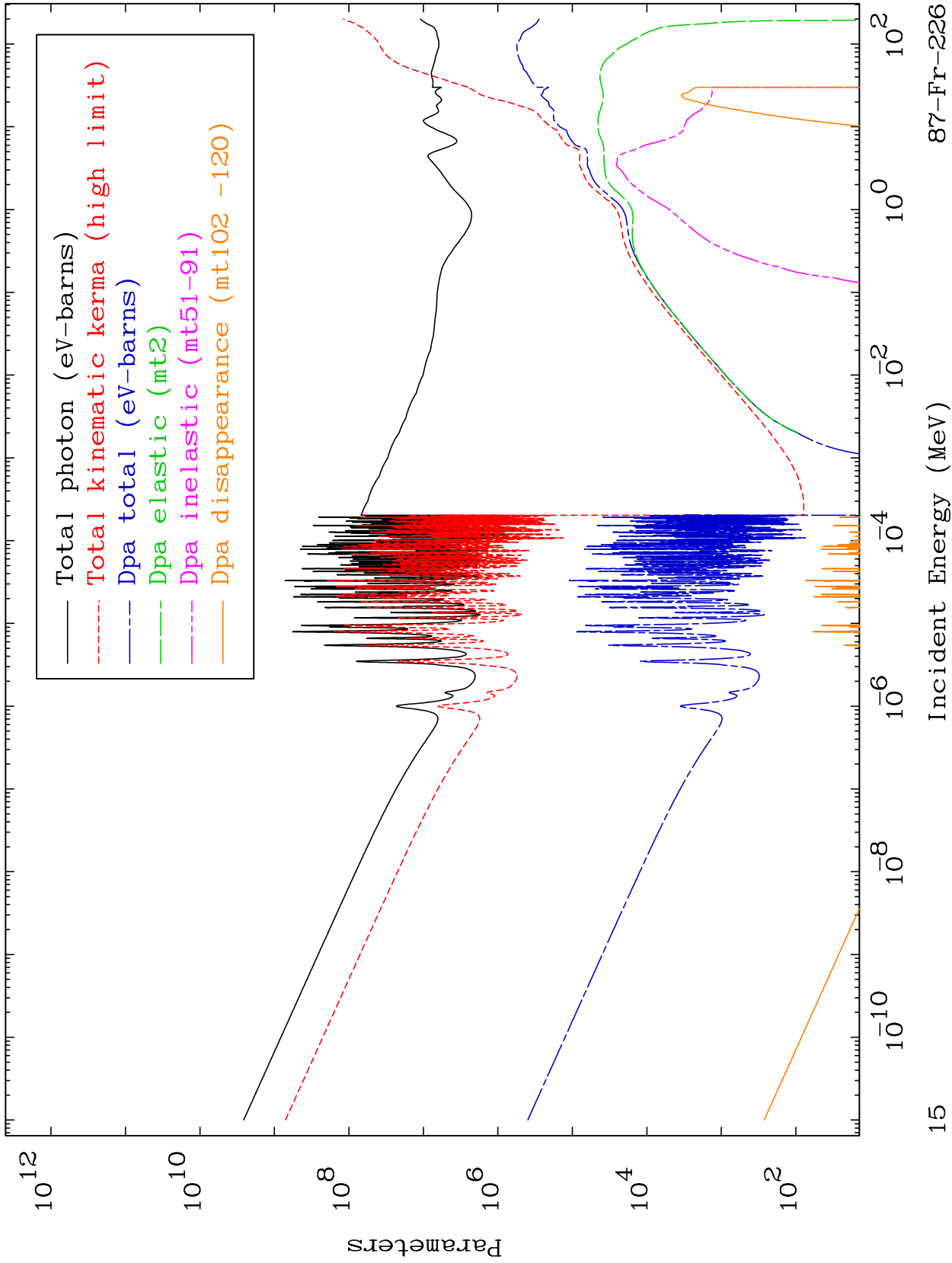


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Incident Energy (MeV)

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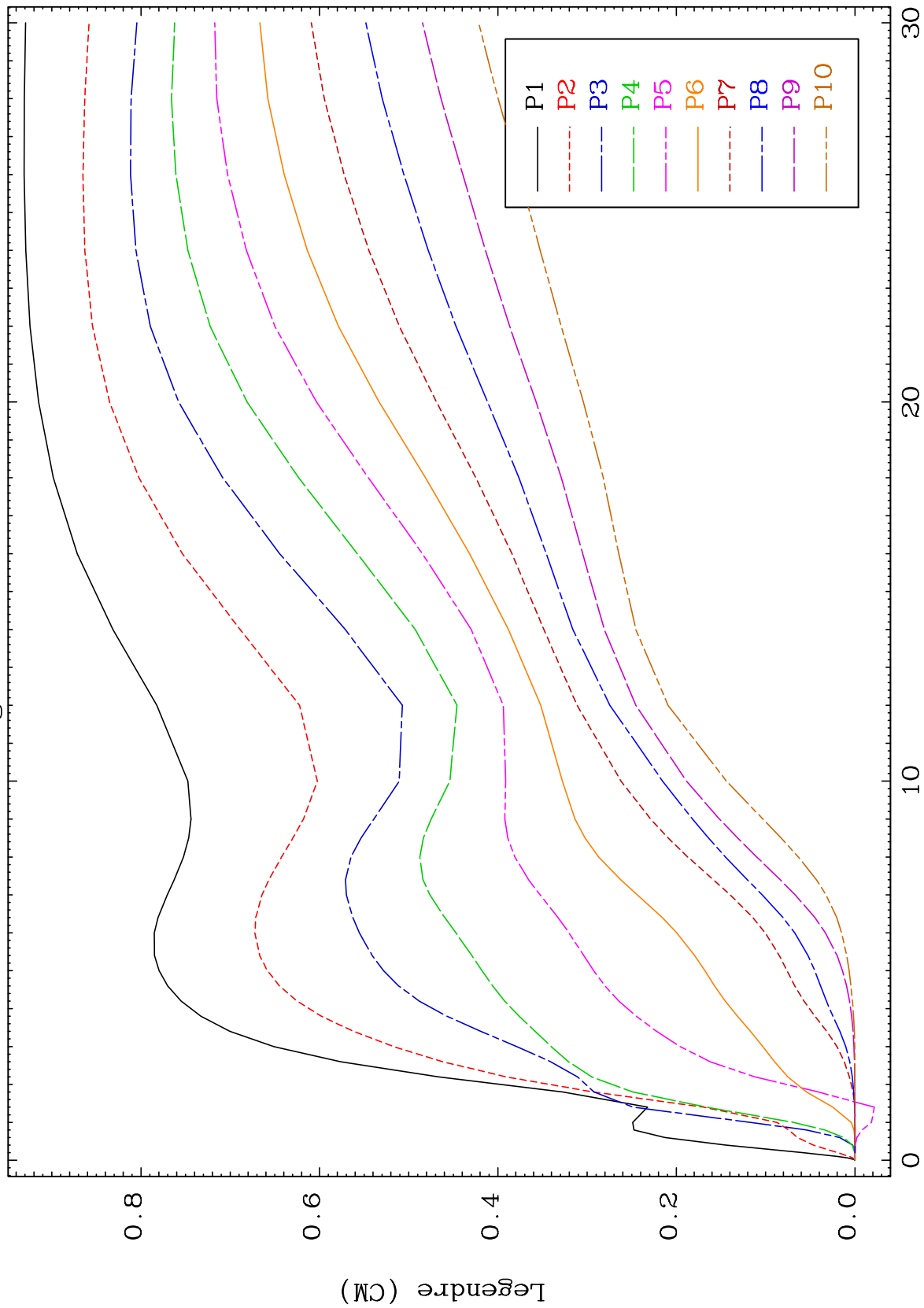




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Elastic Legendre Coefficients

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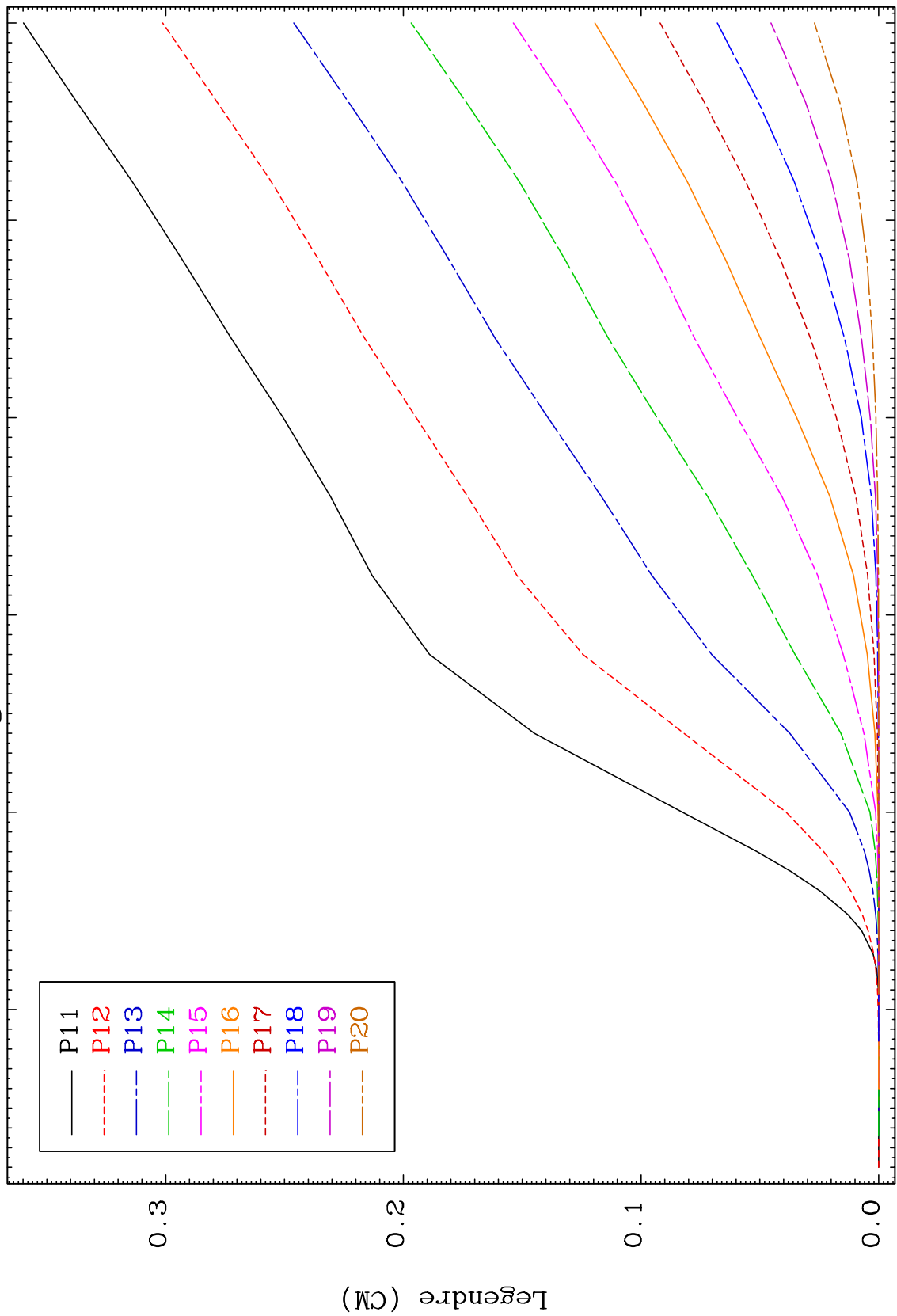
Incident Energy (MeV)

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Elastic Legendre Coefficients

87-Fr-226



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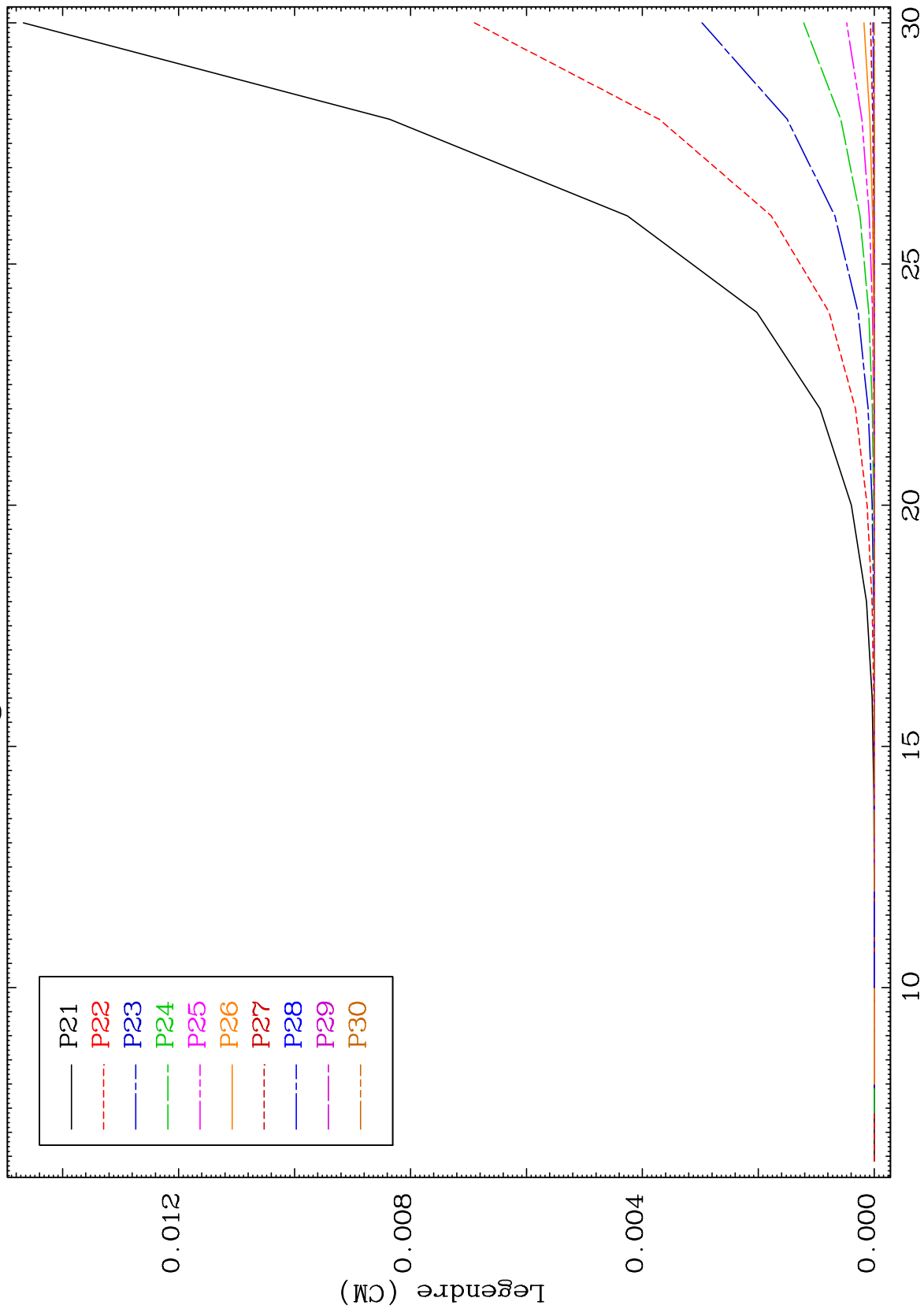
Incident Energy (MeV)

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Elastic Legendre Coefficients

87-Fr-226



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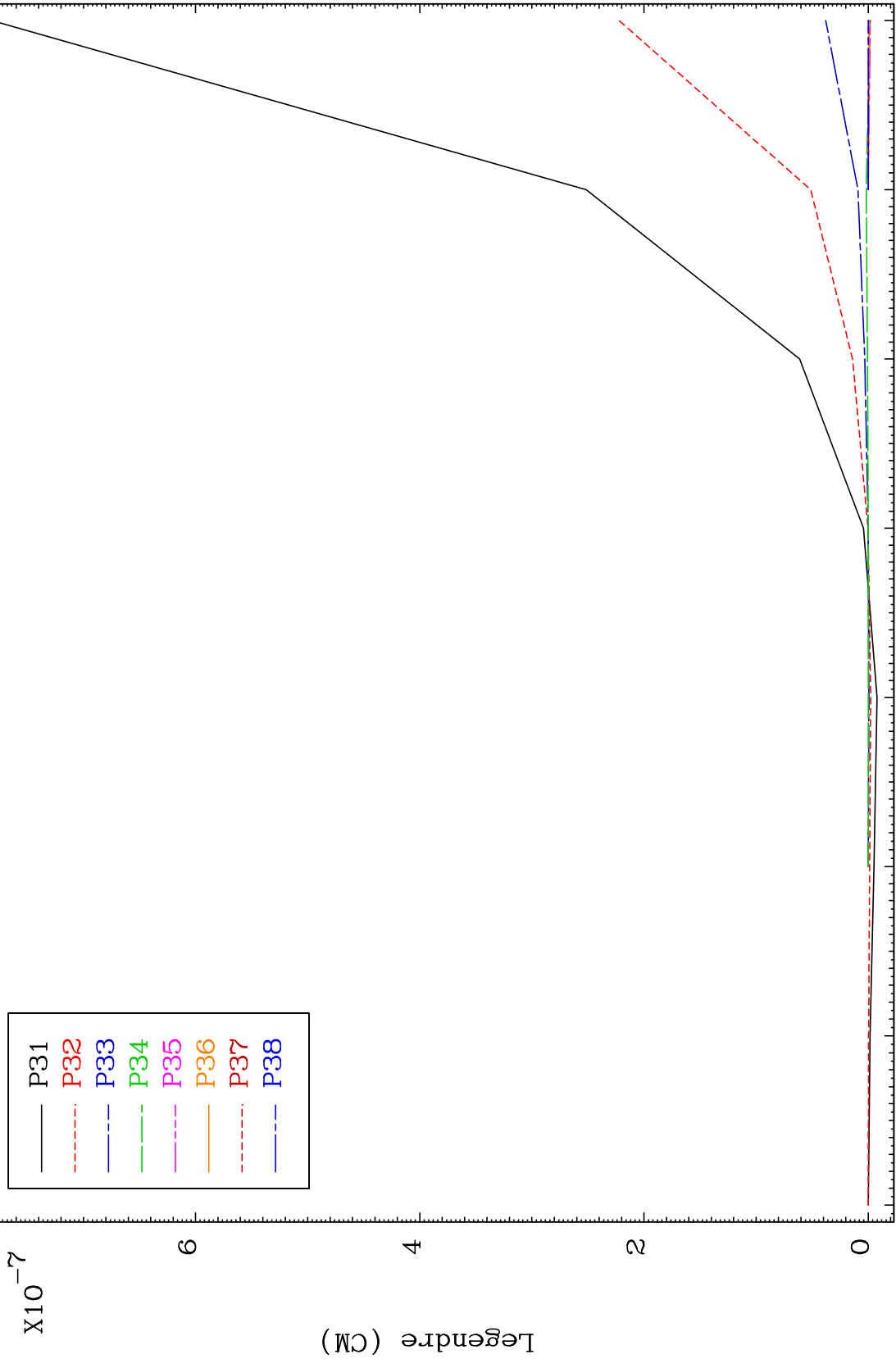
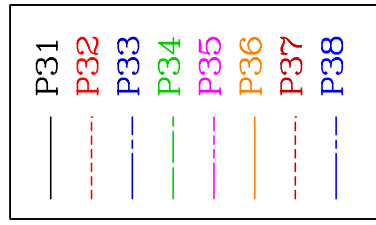
Incident Energy (MeV)

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MAT 8767

Elastic  
Legendre Coefficients

87-Fr-226

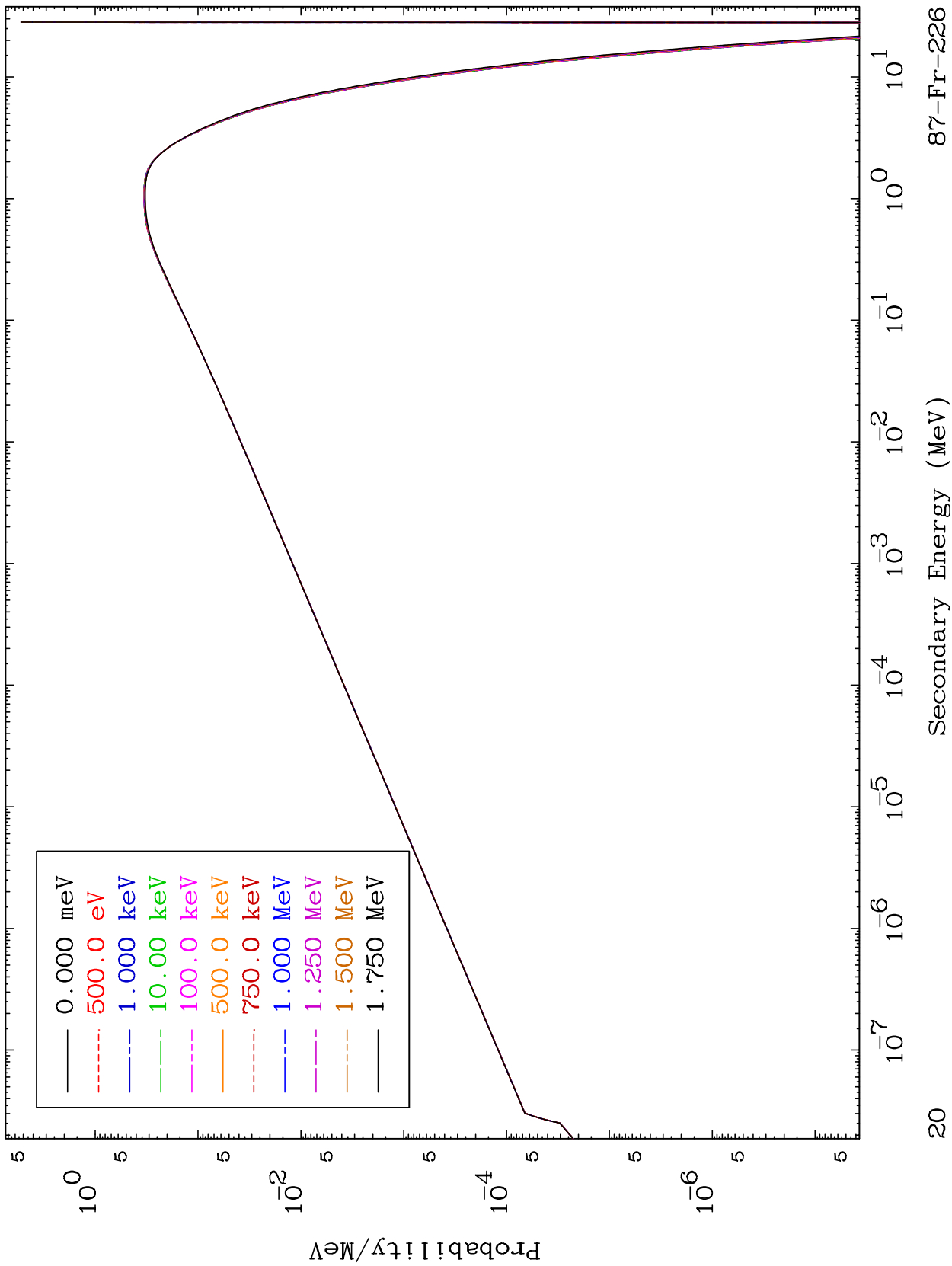


87-Fr-226

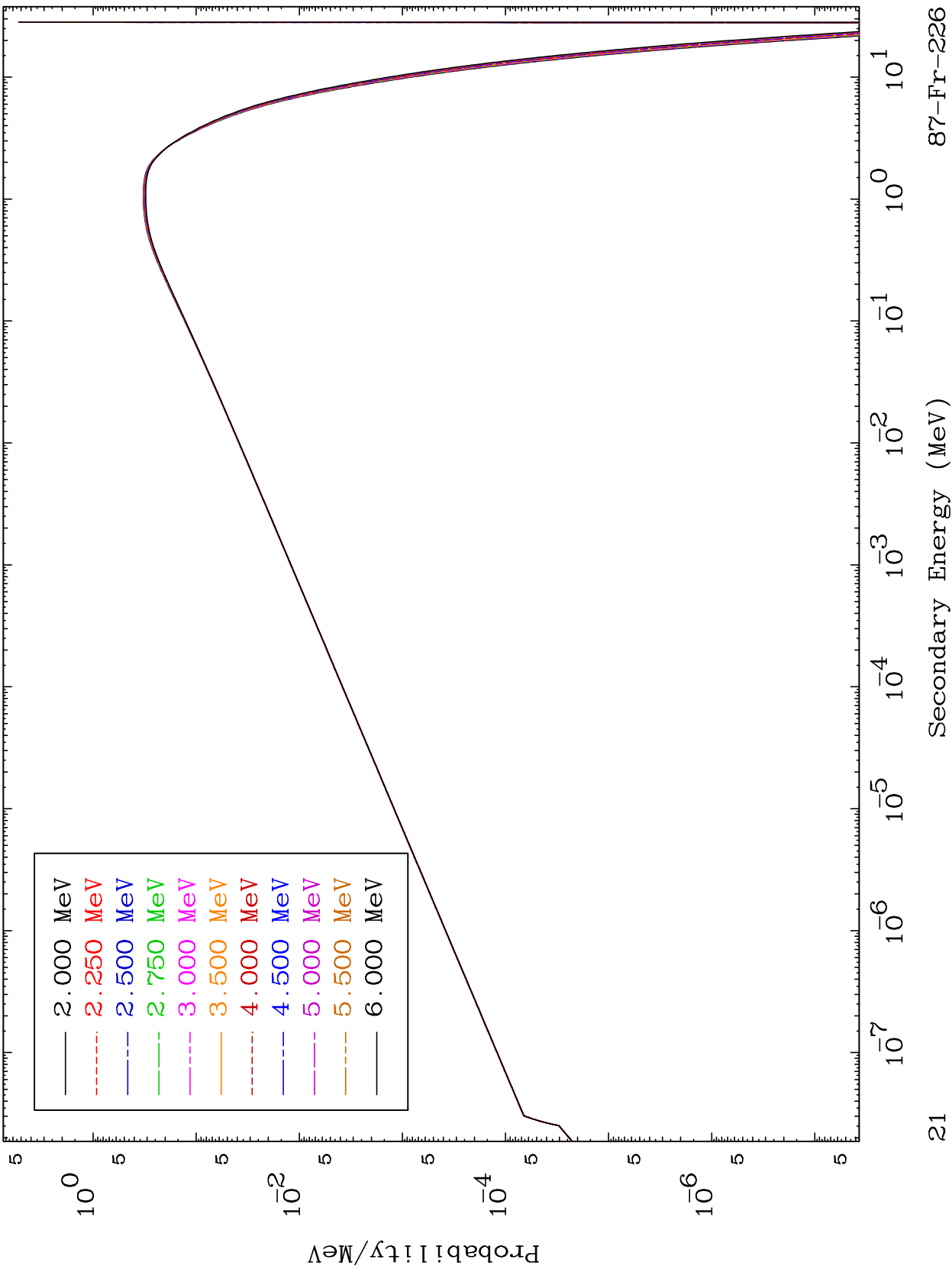
Incident Energy (MeV)

19

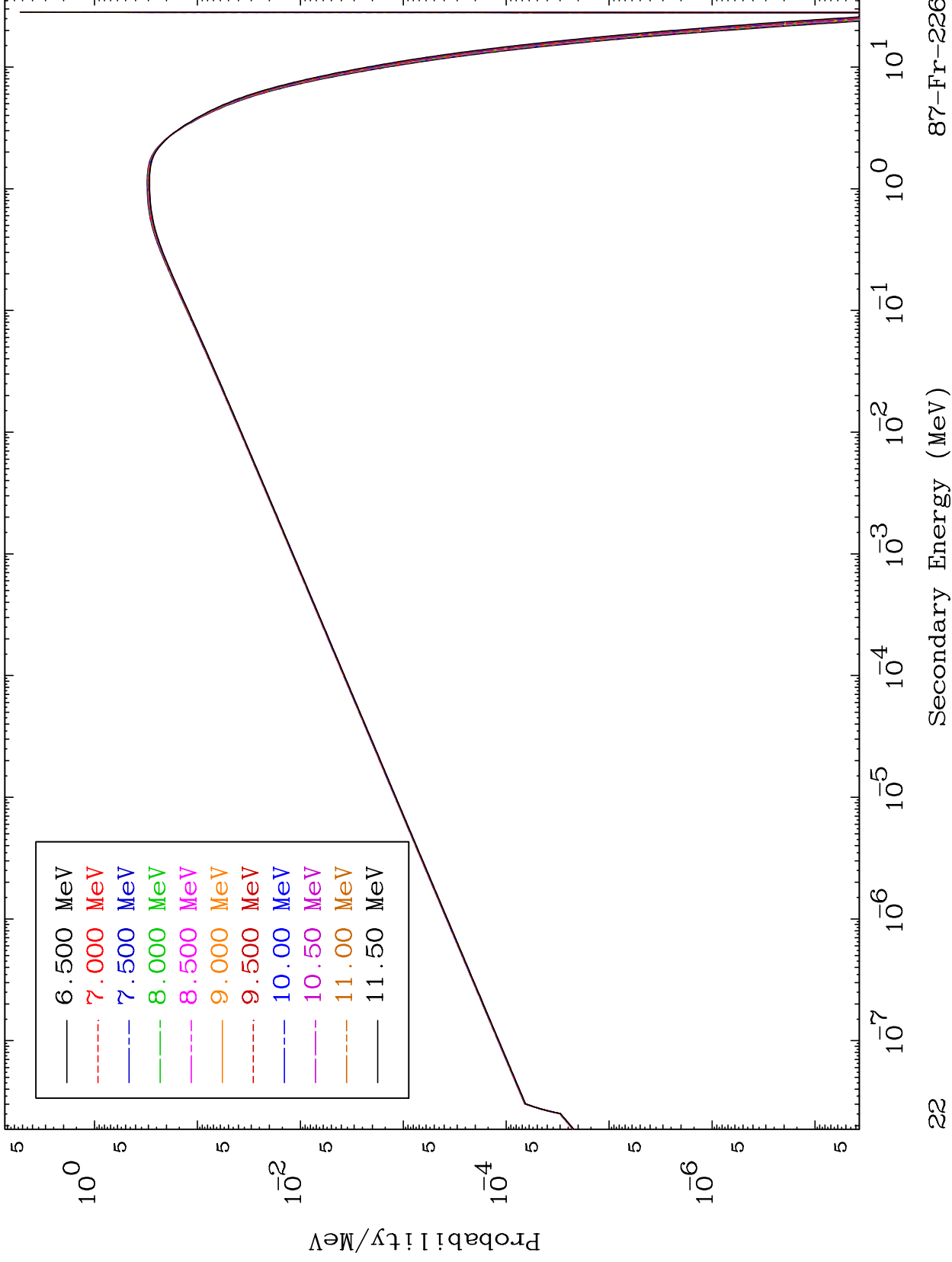
Fission Energy Distribution



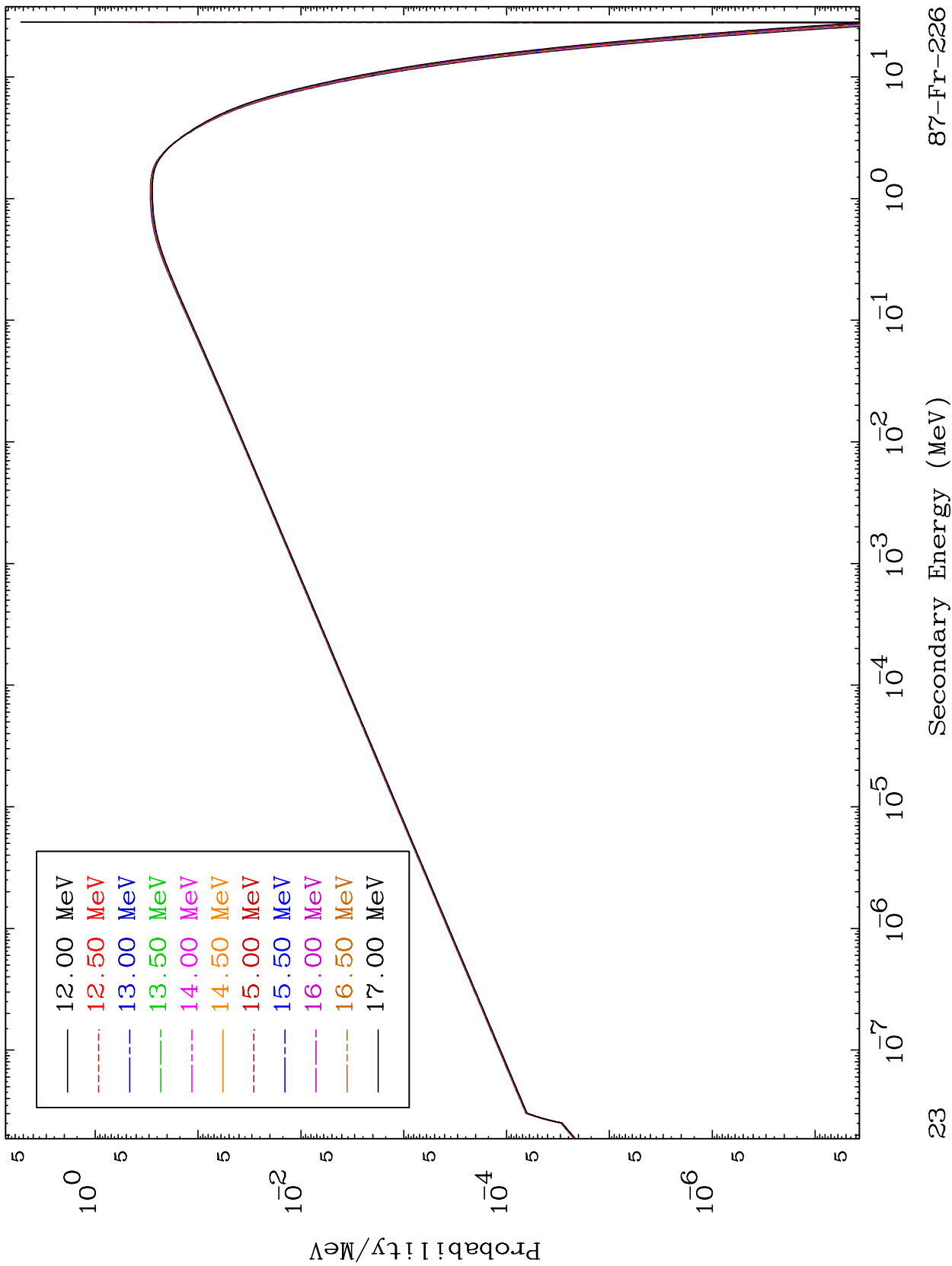
Fission Energy Distribution



Fission  
Energy Distribution



Fission  
Energy Distribution





Fission  
Energy Distribution

