

Program EVALPLOT  
(Version 2015-2)

by

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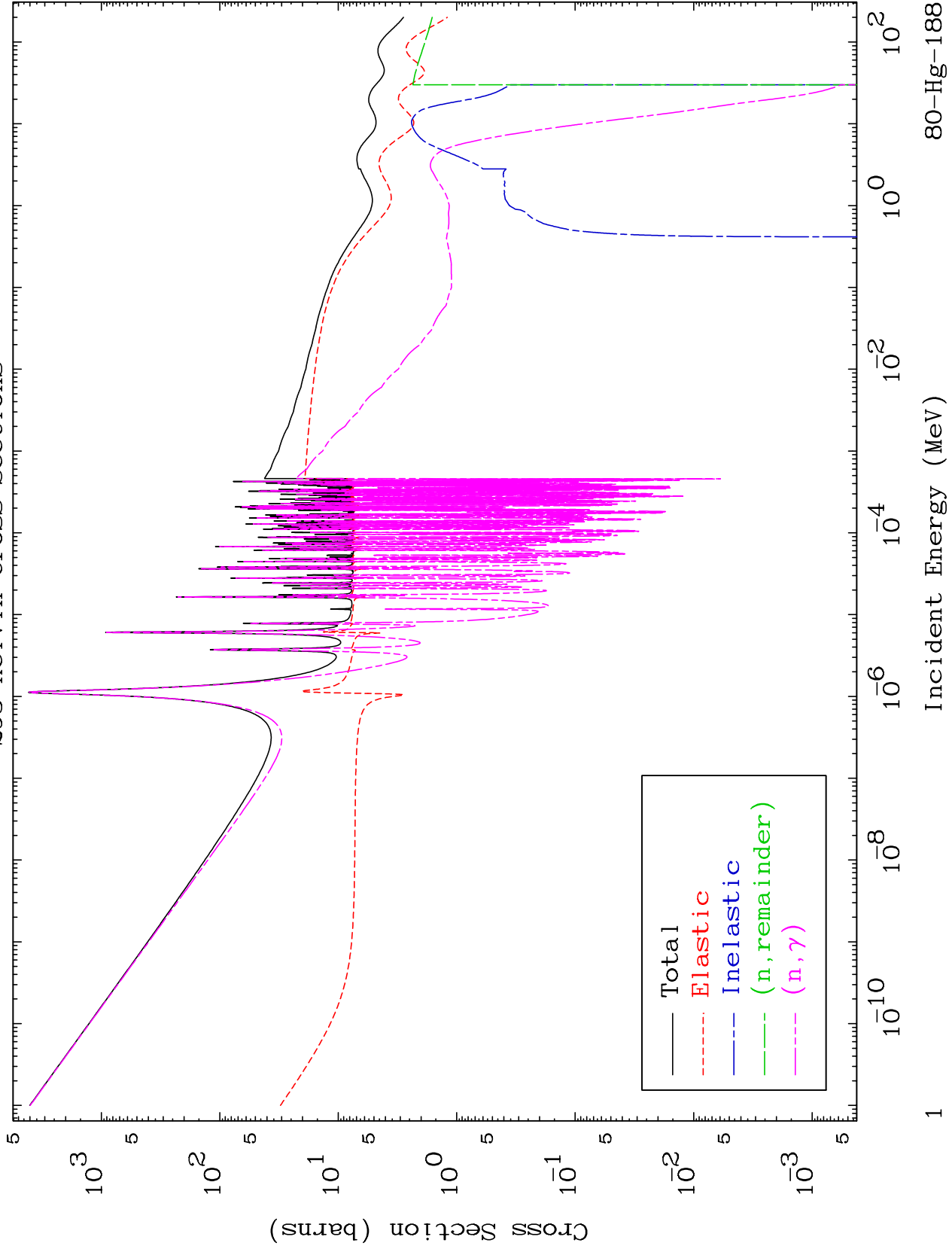
Press Mouse Button to Start

MAT 8001

Major

293 Kelvin Cross Sections

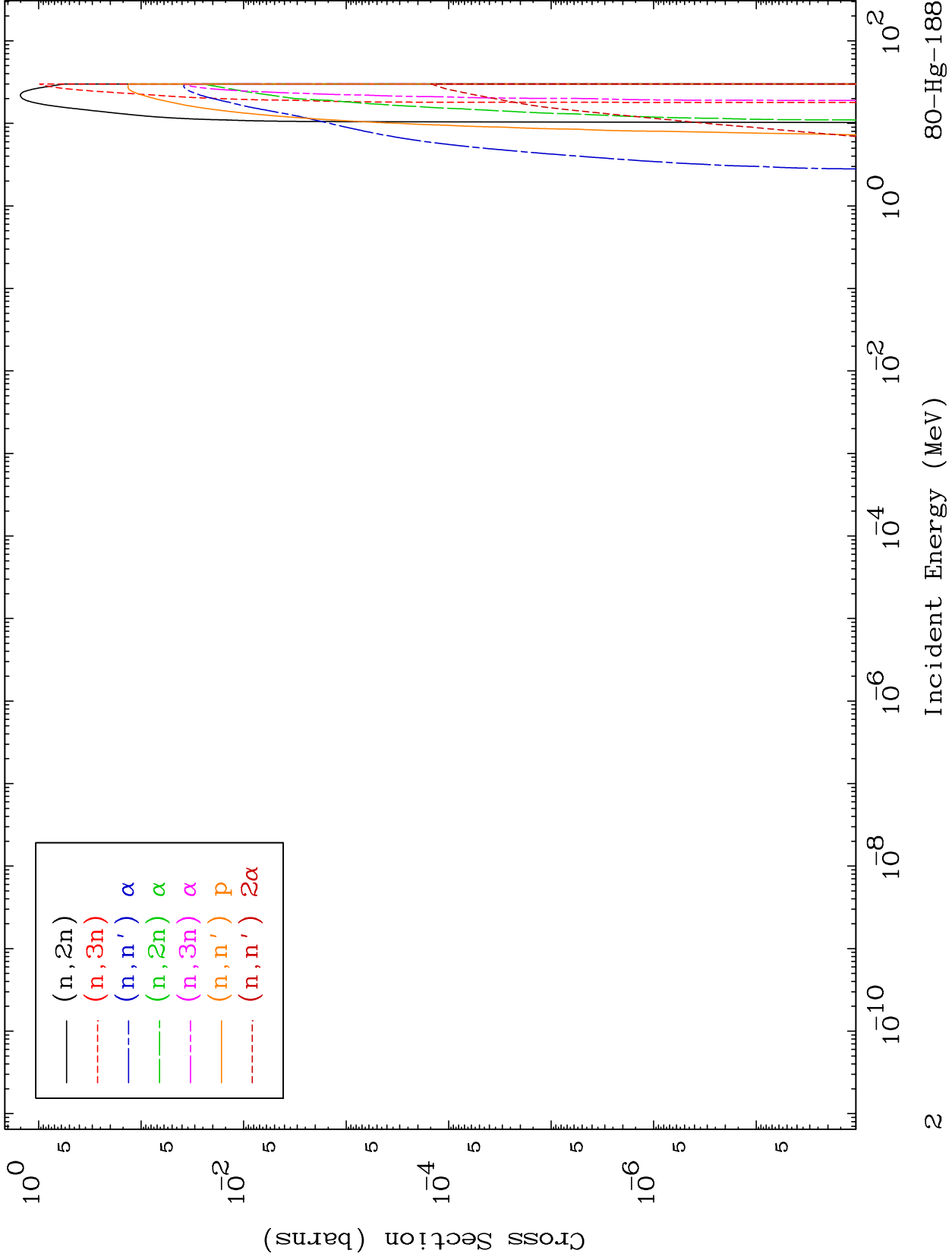
80-Hg-188



MAT 8001

Neutron Production  
293 Kelvin Cross Sections

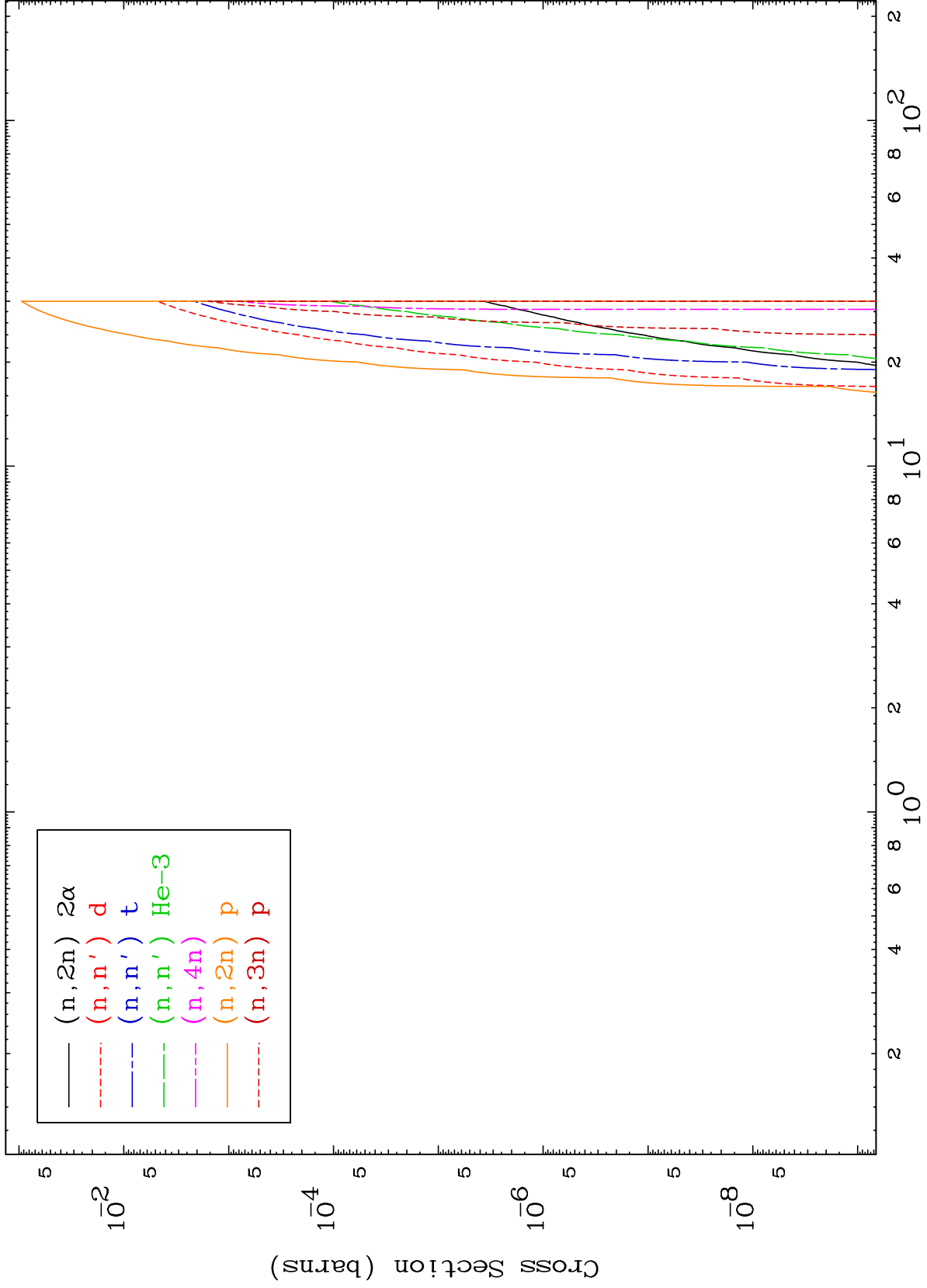
80-Hg-188



MAT 8001

Neutron Production  
293 Kelvin Cross Sections

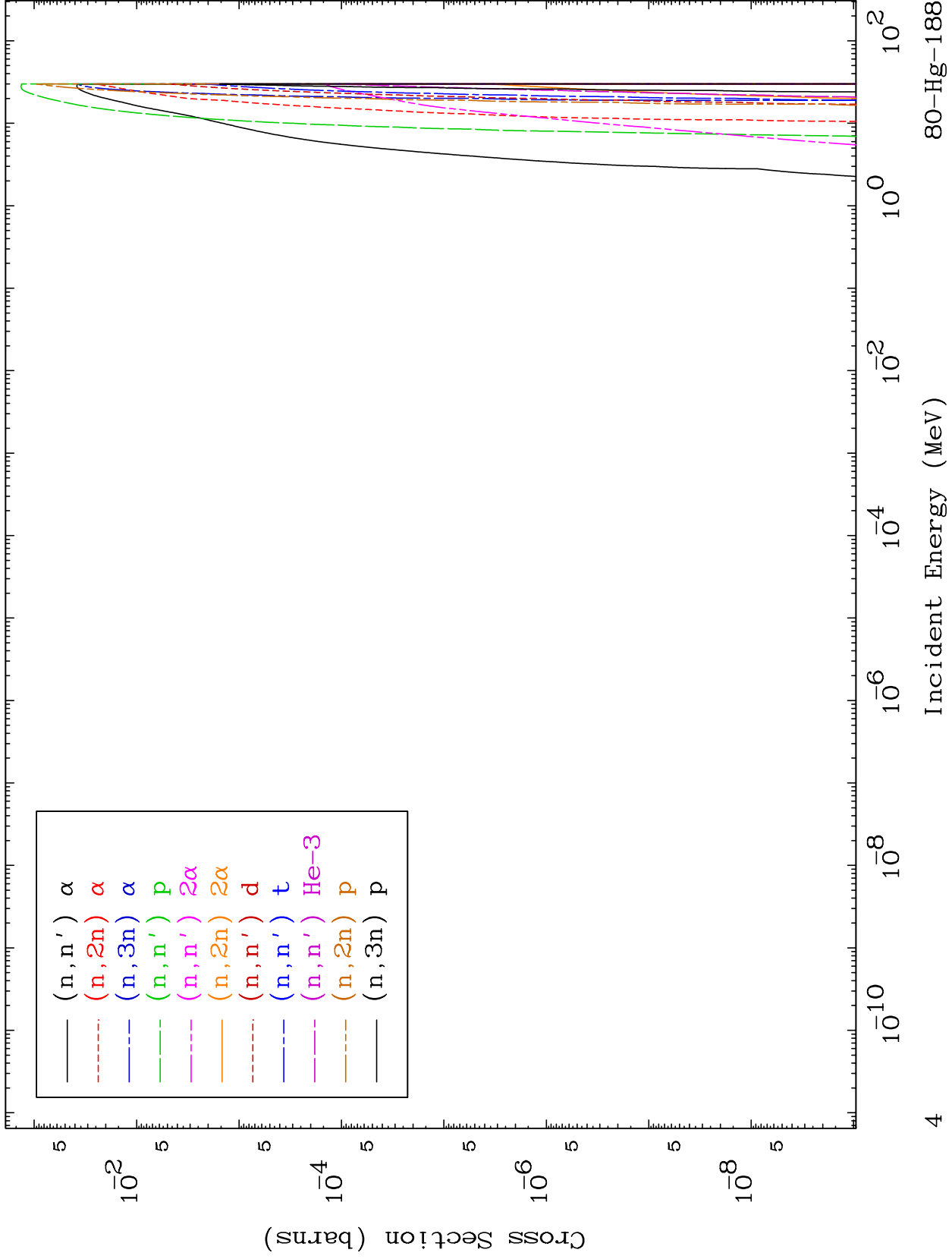
80-Hg-188



MAT 8001

Charged Particle  
293 Kelvin Cross Sections

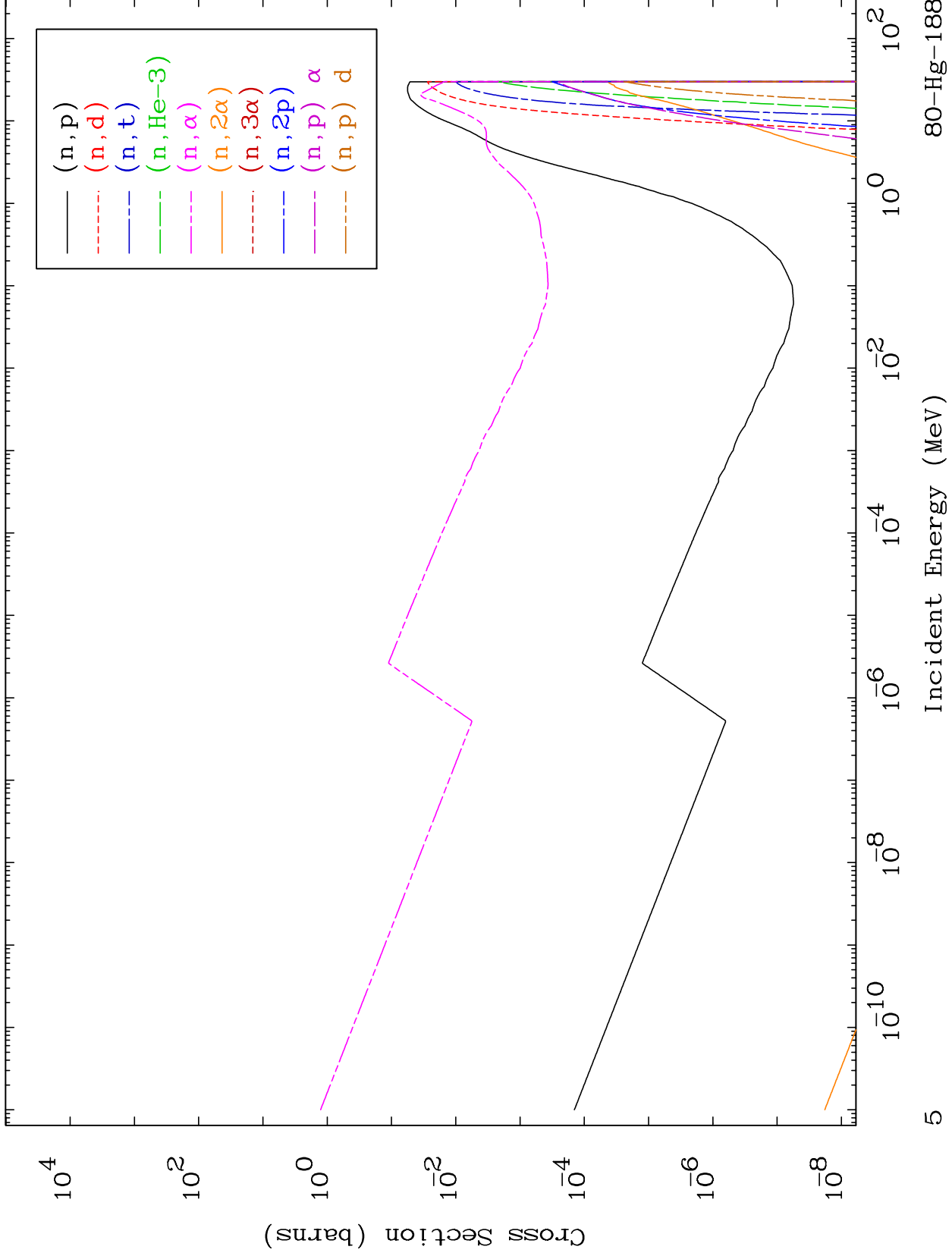
80-Hg-188



MAT 8001

Charged Particle  
293 Kelvin Cross Sections

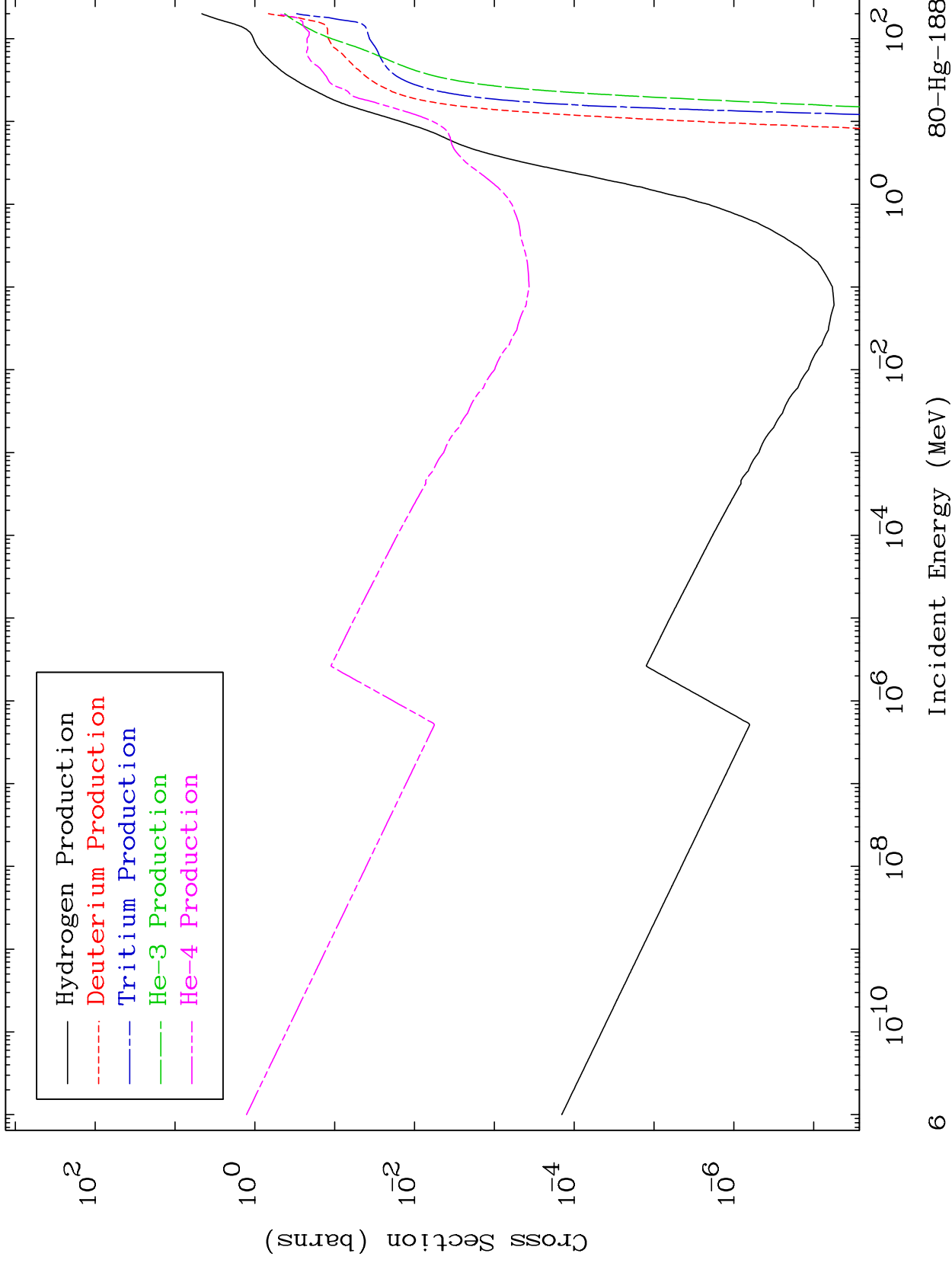
80-Hg-188



MAT 8001

Particle Production  
293 Kelvin Cross Sections

80-Hg-188

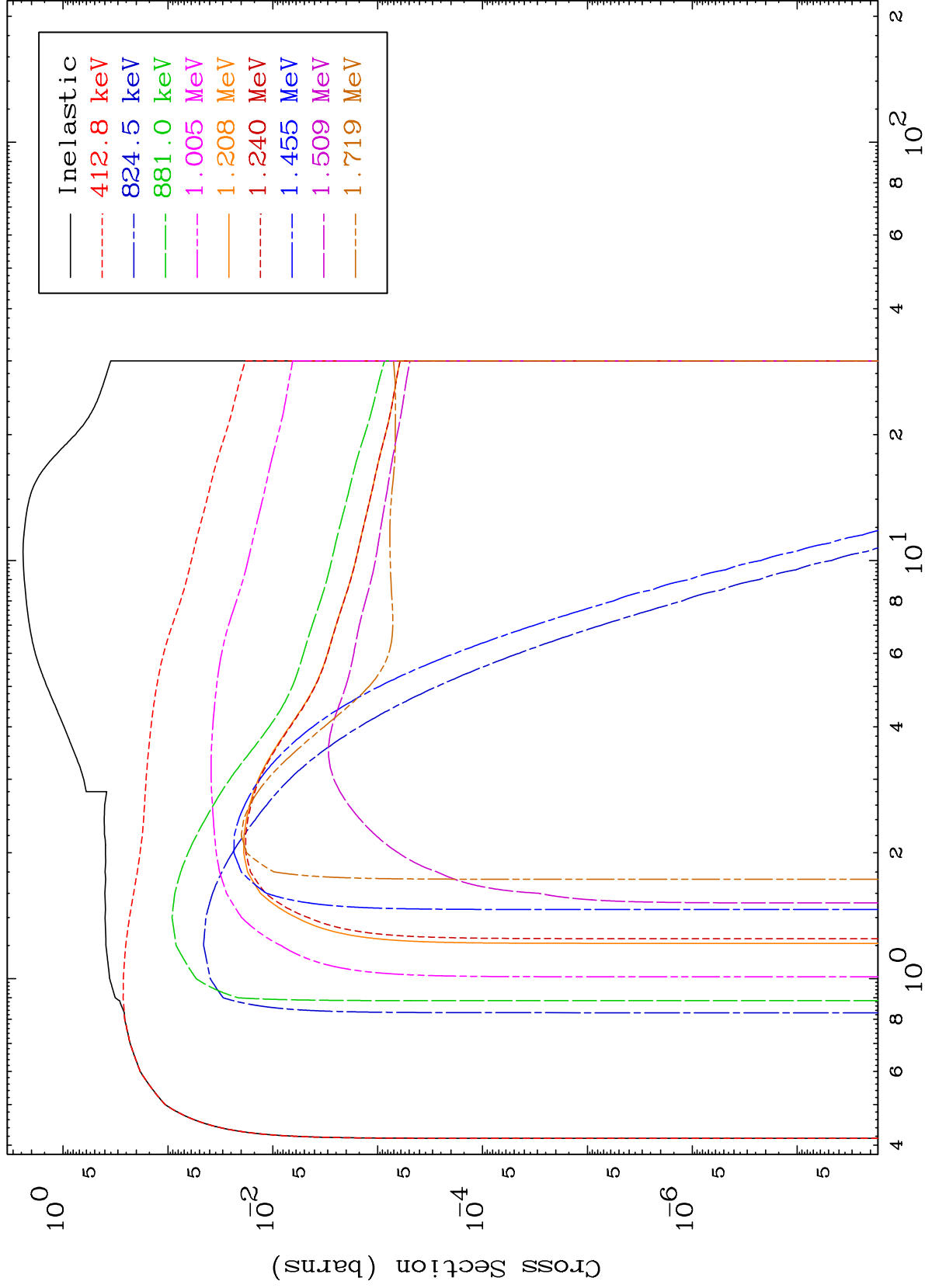


MAT 8001

(n,n') Level

80-Hg-188

293 Kelvin Cross Sections



7

Incident Energy (MeV)

80-Hg-188

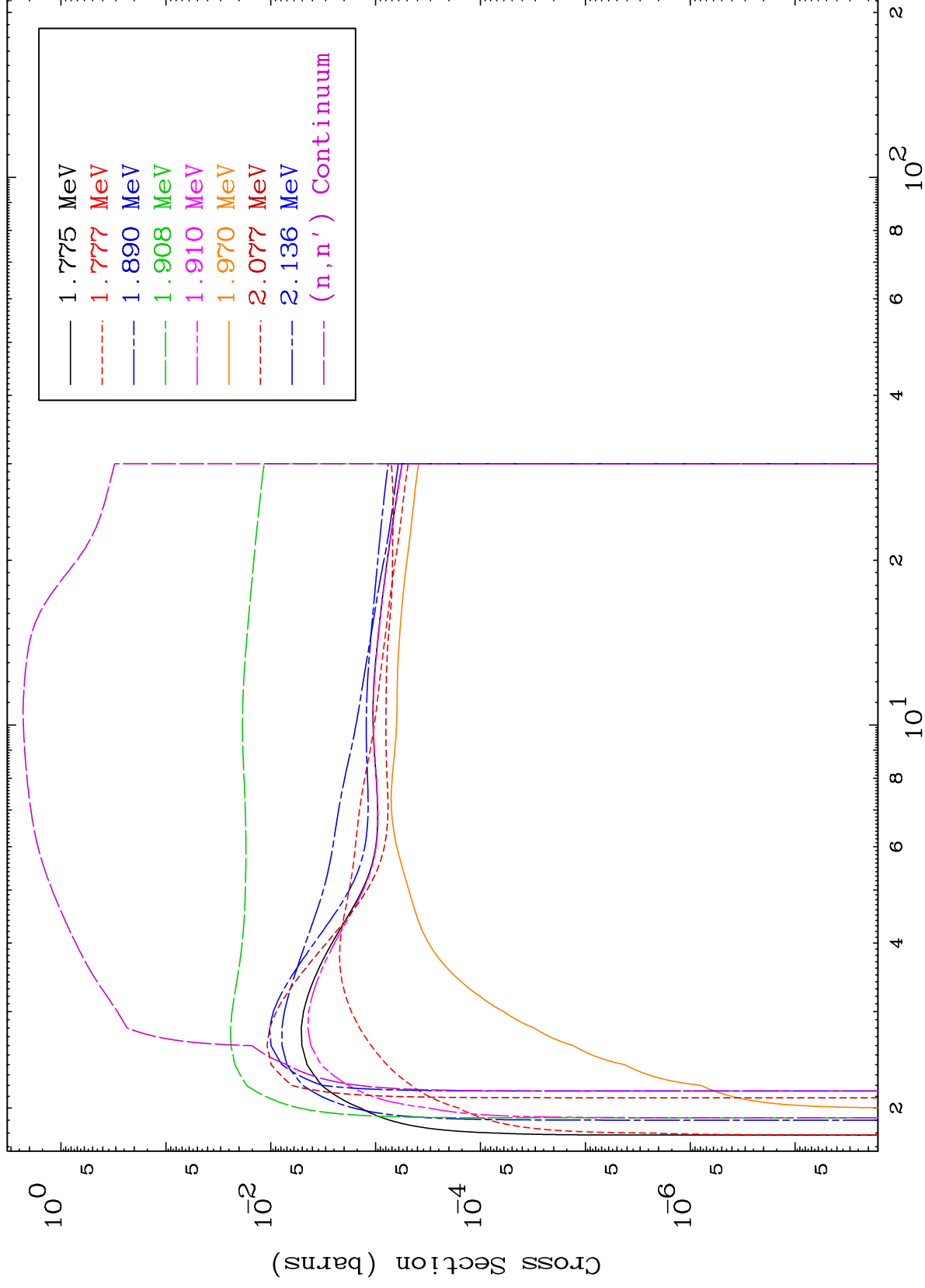


MAT 8001

(n,n') Level

293 Kelvin Cross Sections

80-Hg-188



8

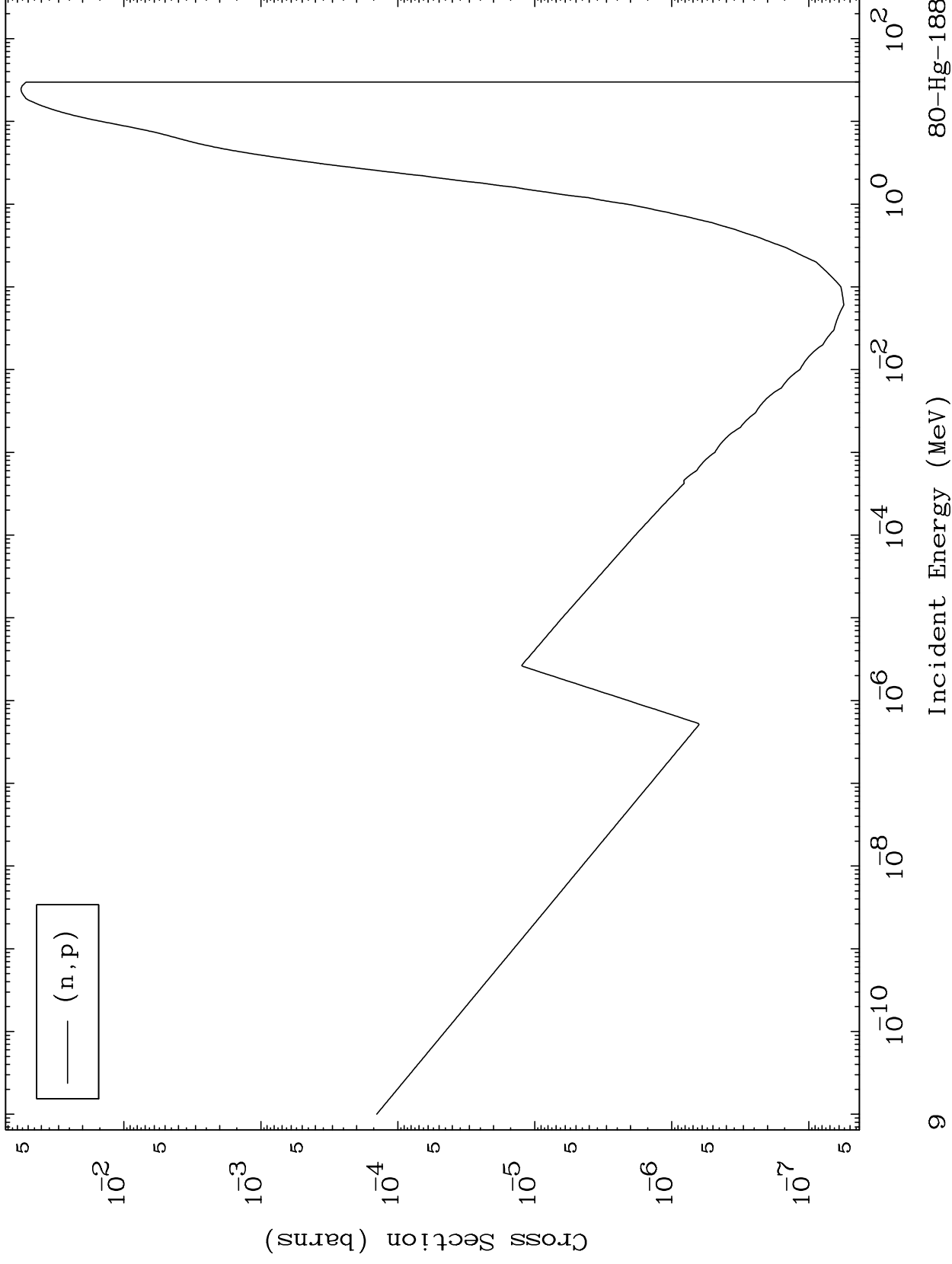
Incident Energy (MeV)

80-Hg-188

MAT 8001

(n,p) Levels  
293 Kelvin Cross Sections

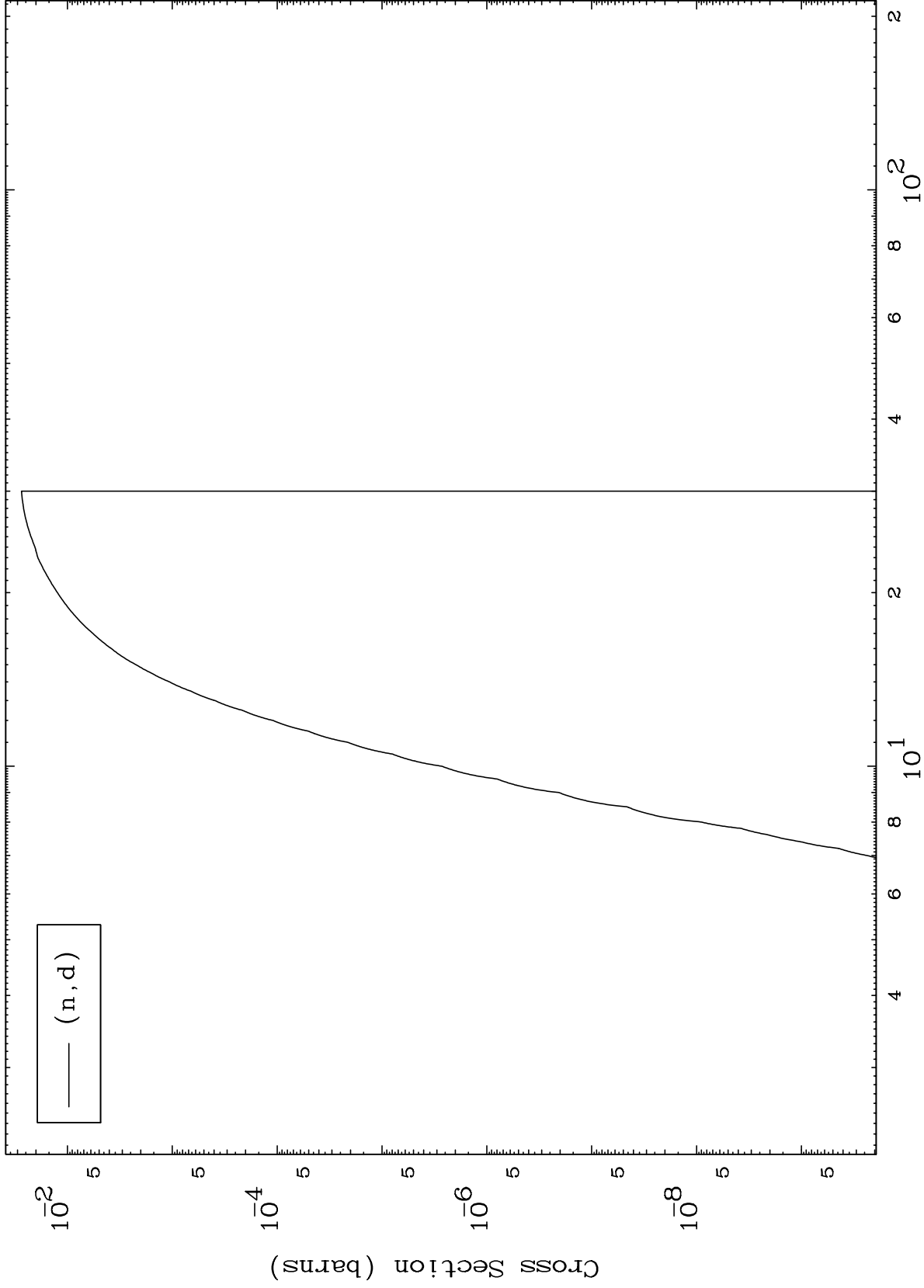
80-Hg-188



MAT 8001

(n,d) Levels  
293 Kelvin Cross Sections

80-Hg-188



10

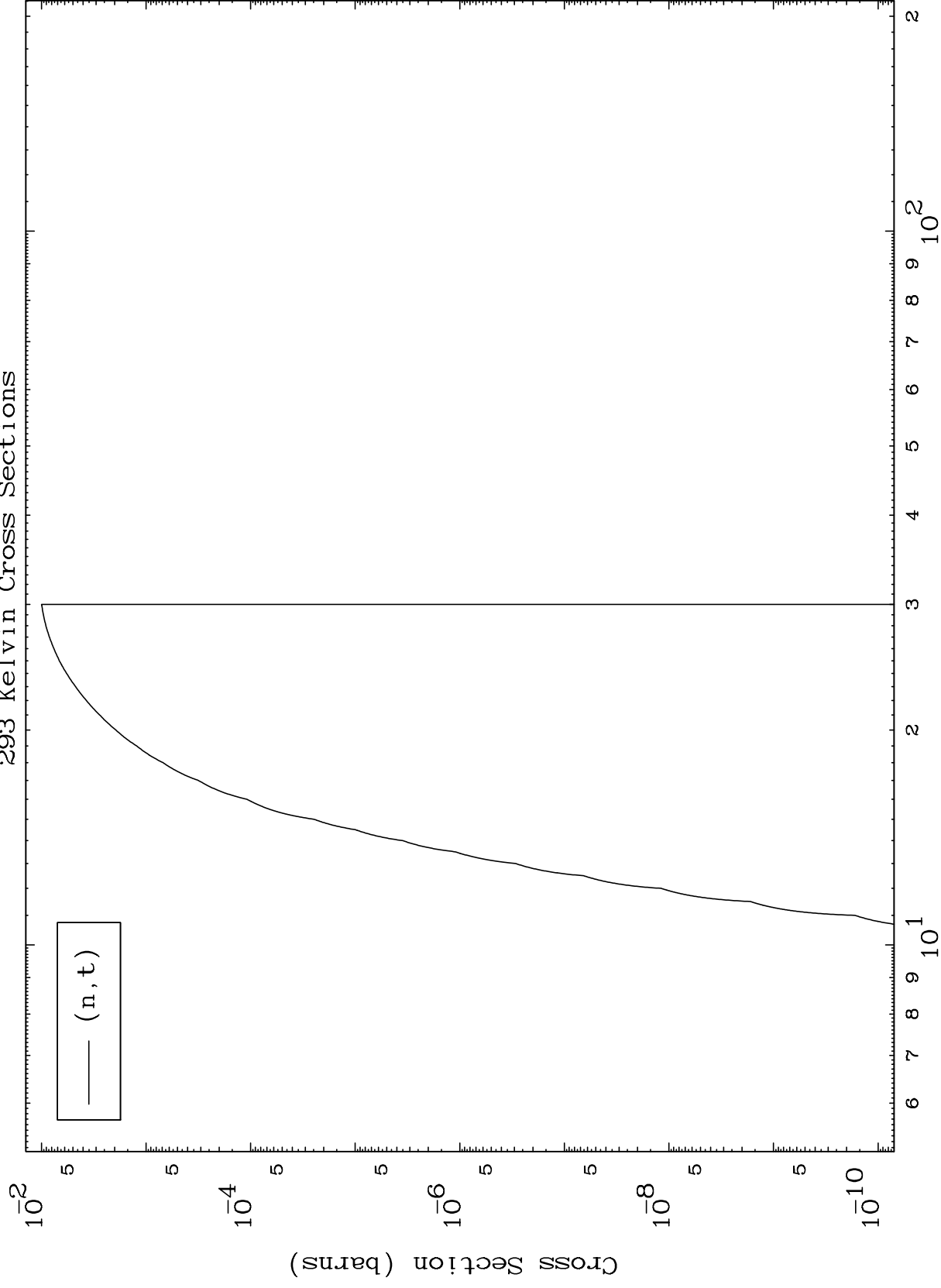
Incident Energy (MeV)

80-Hg-188

MAT 8001

80-Hg-188

(n,t) Levels  
293 Kelvin Cross Sections



11

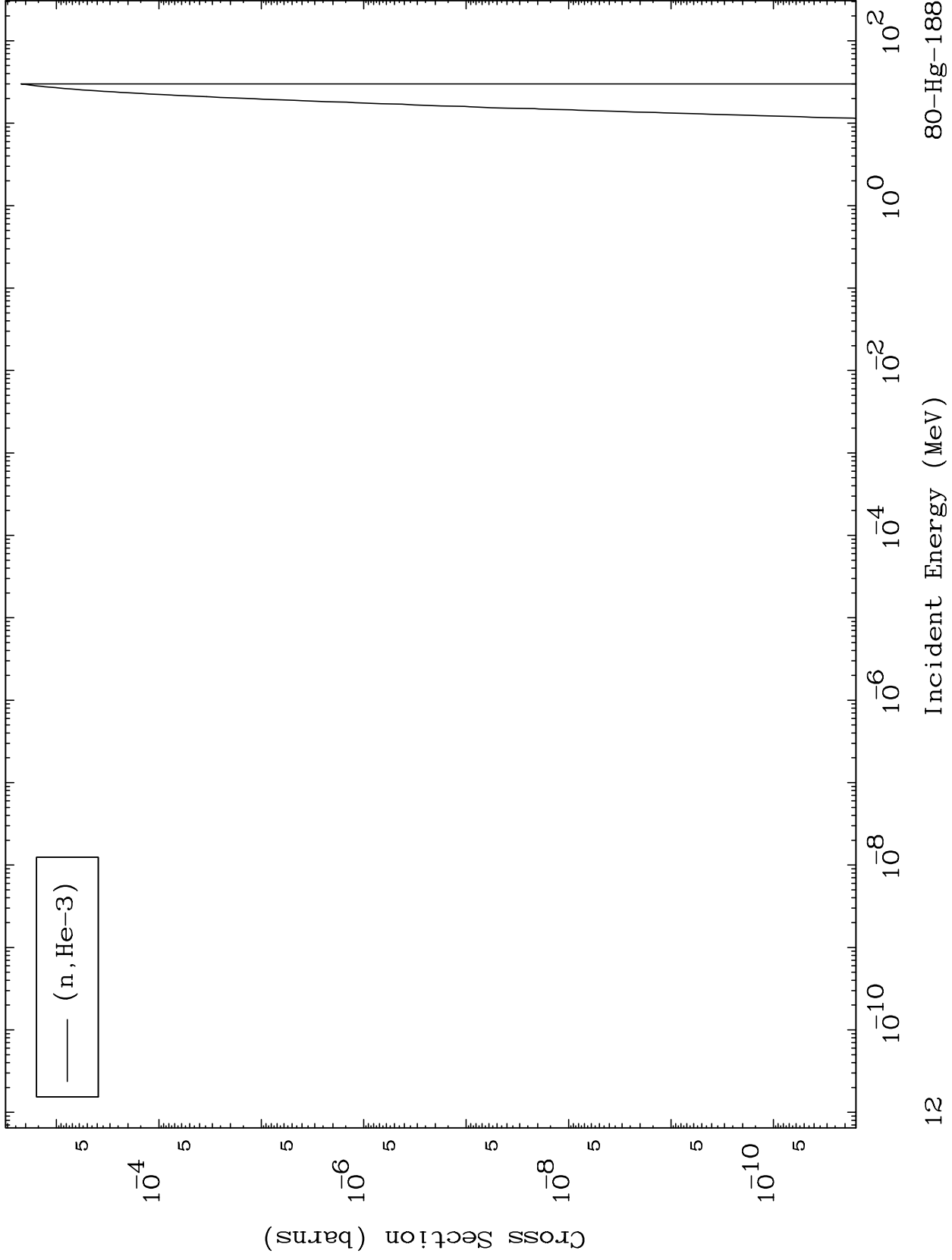
80-Hg-188

Incident Energy (MeV)

MAT 8001

(n,He3) Levels  
293 Kelvin Cross Sections

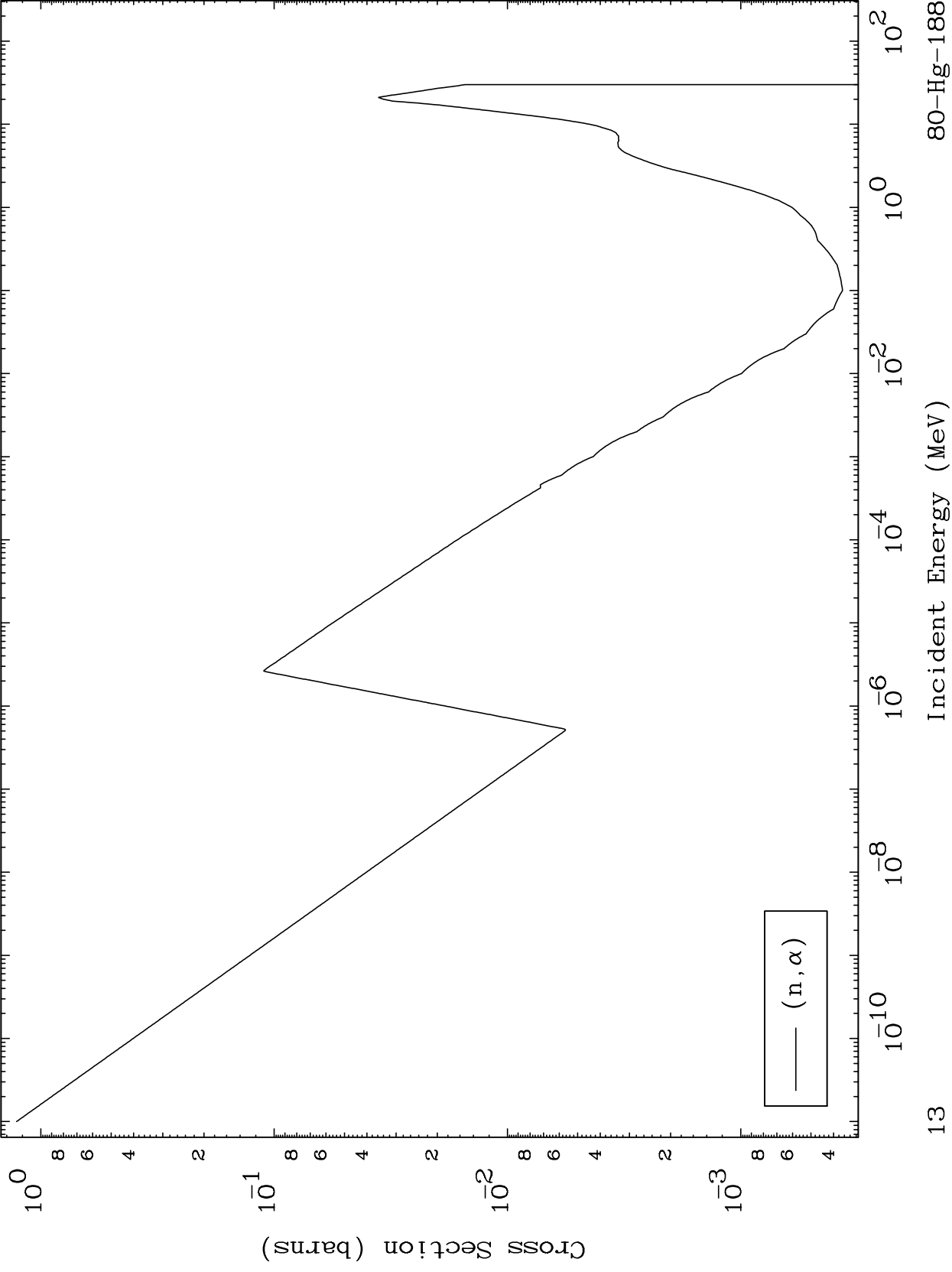
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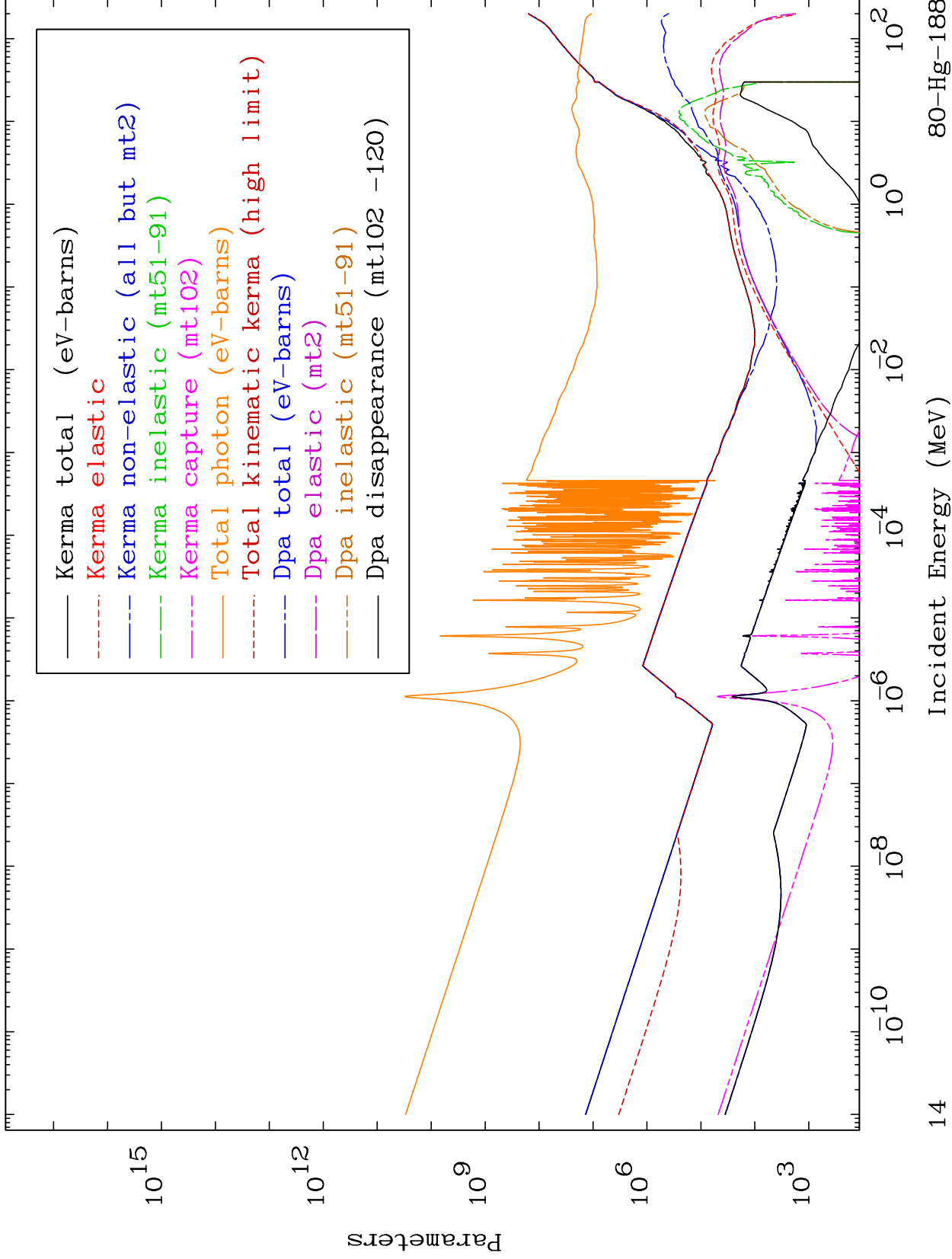


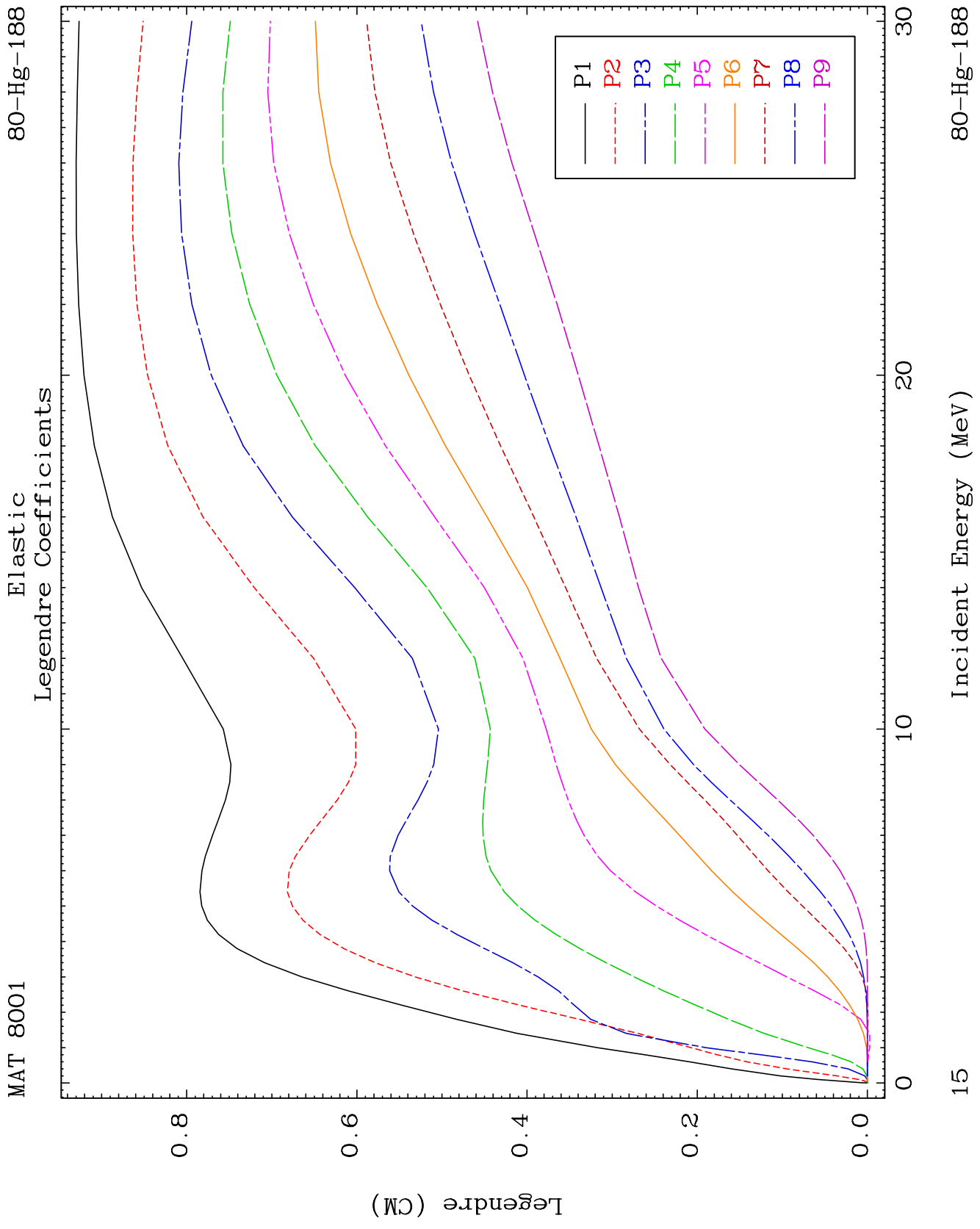
MAT 8001

(n,  $\alpha$ ) Levels  
293 Kelvin Cross Sections

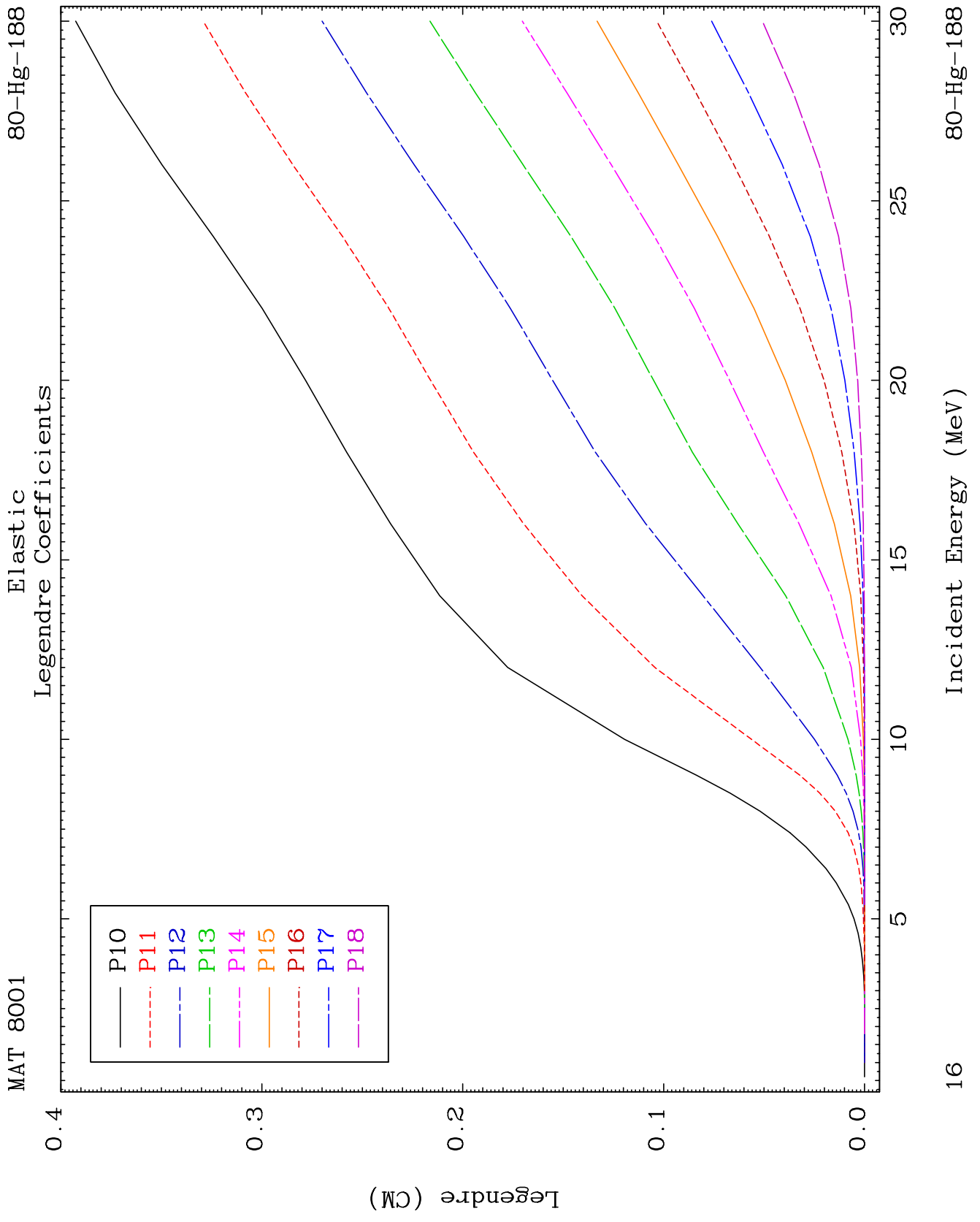
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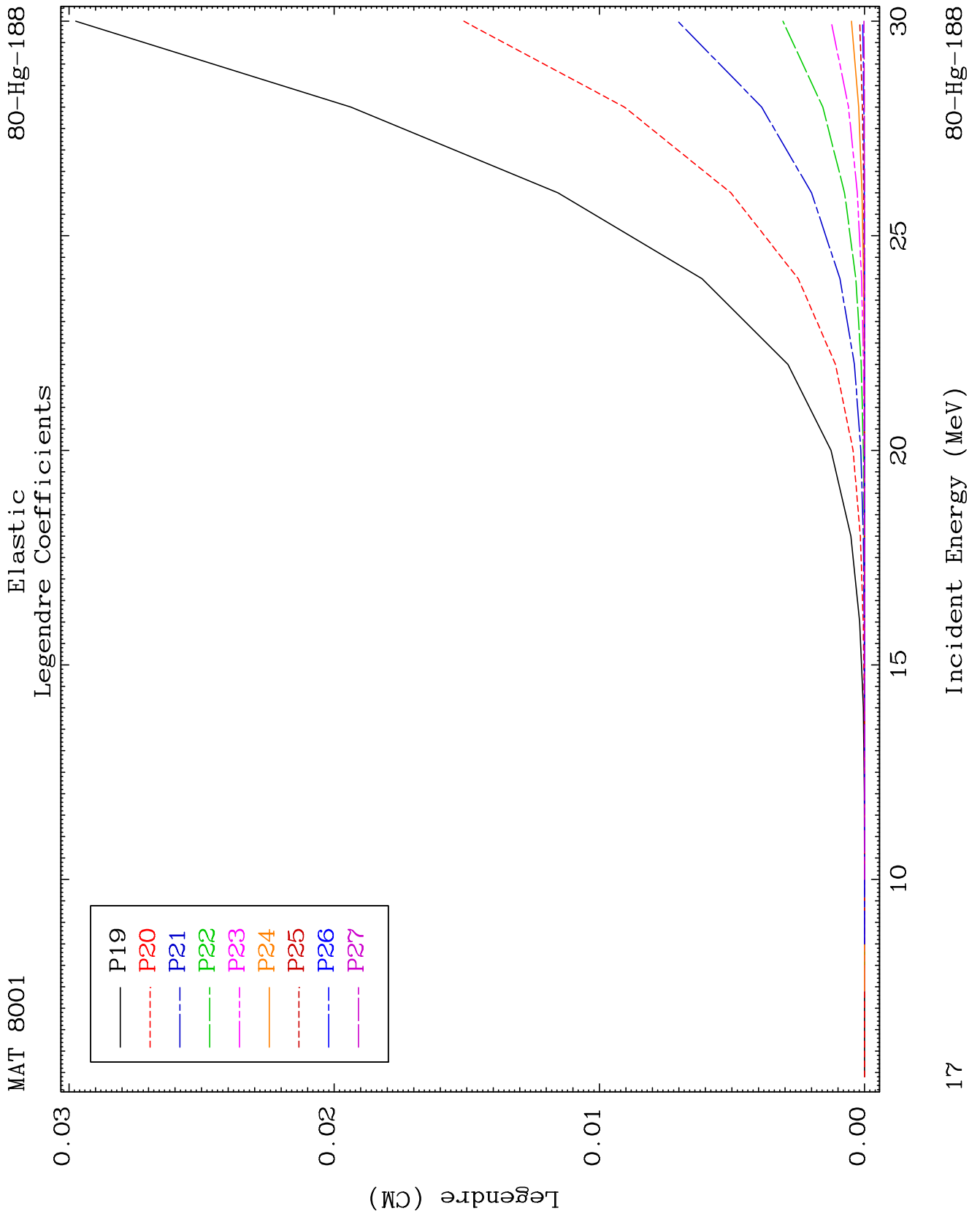


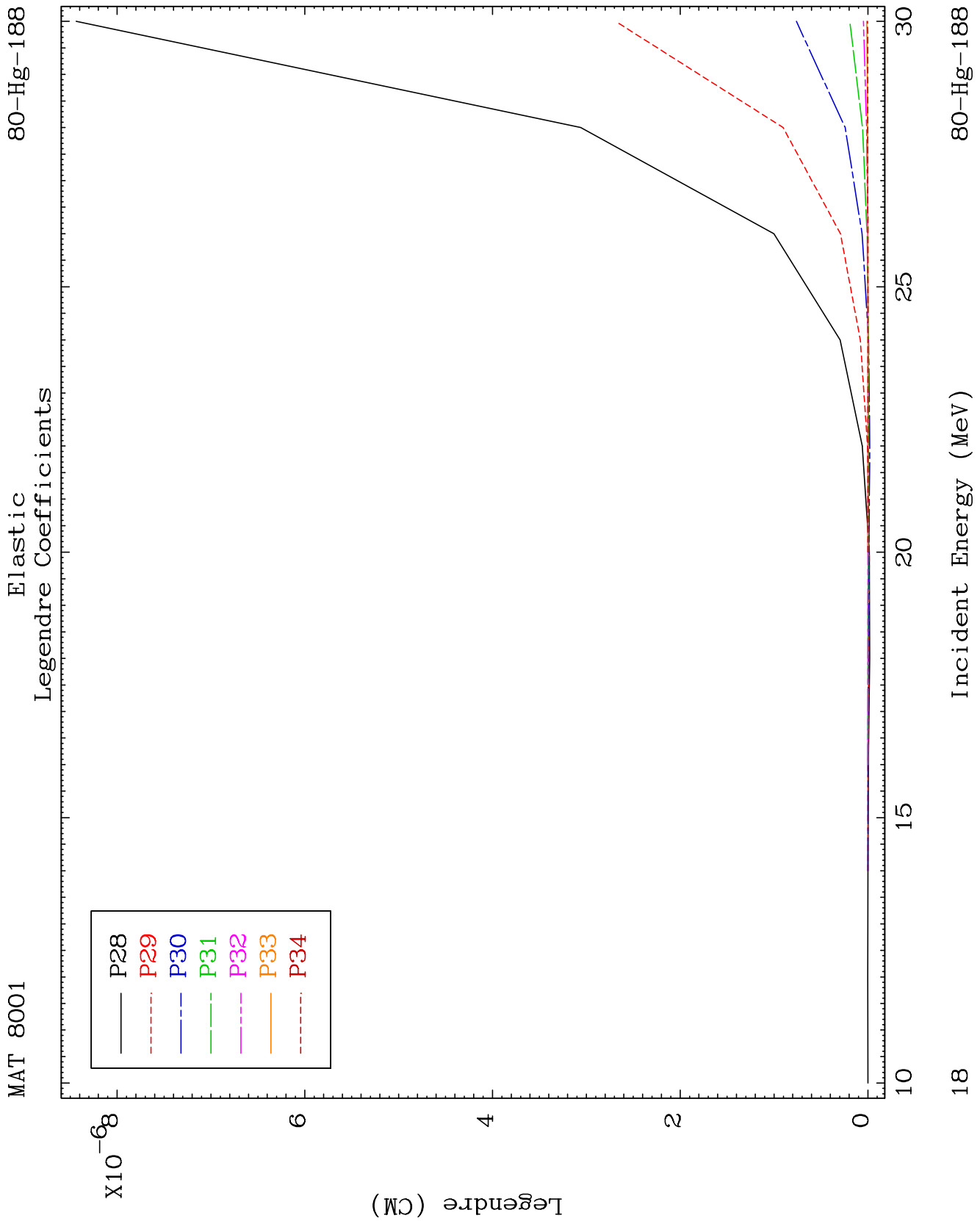


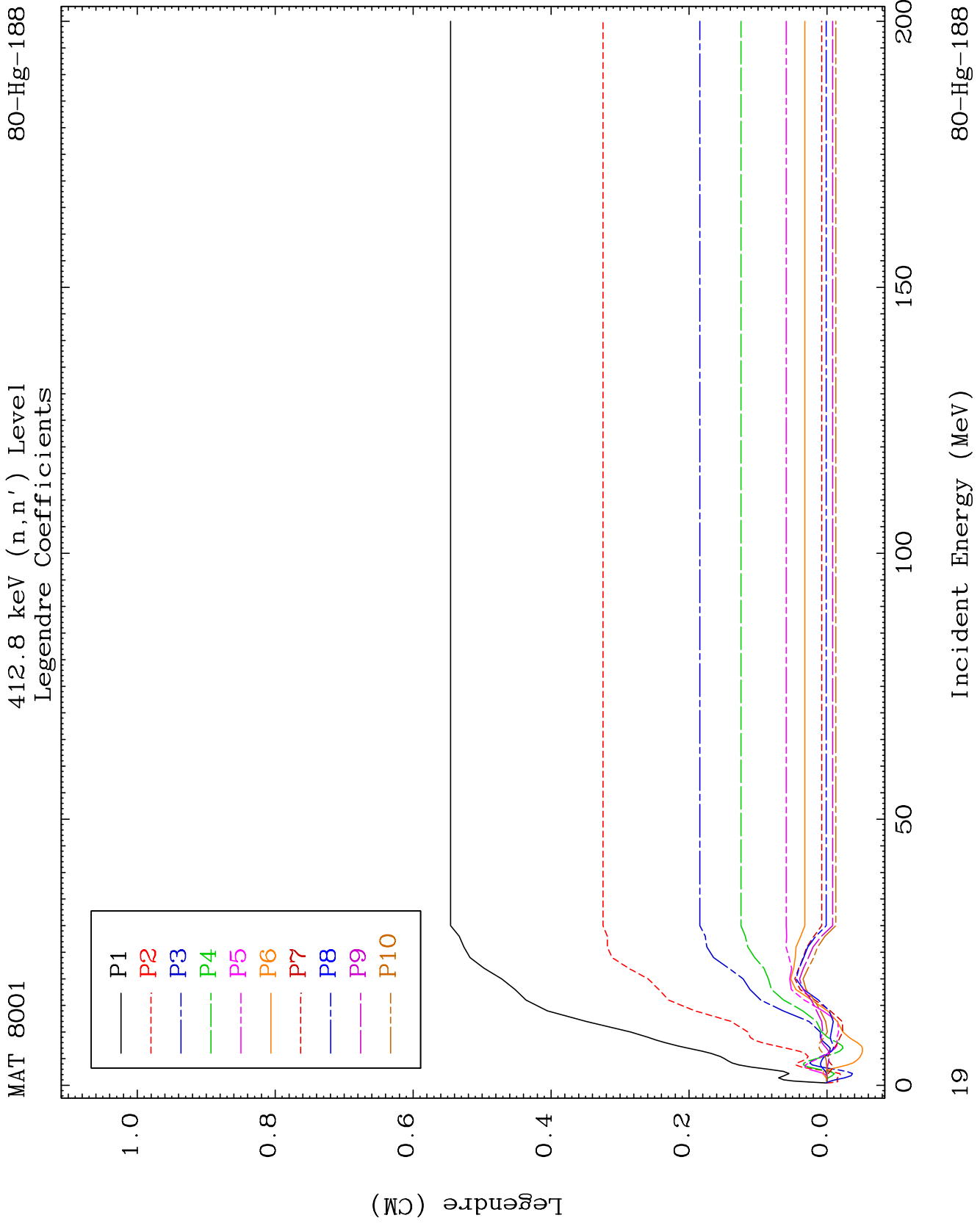


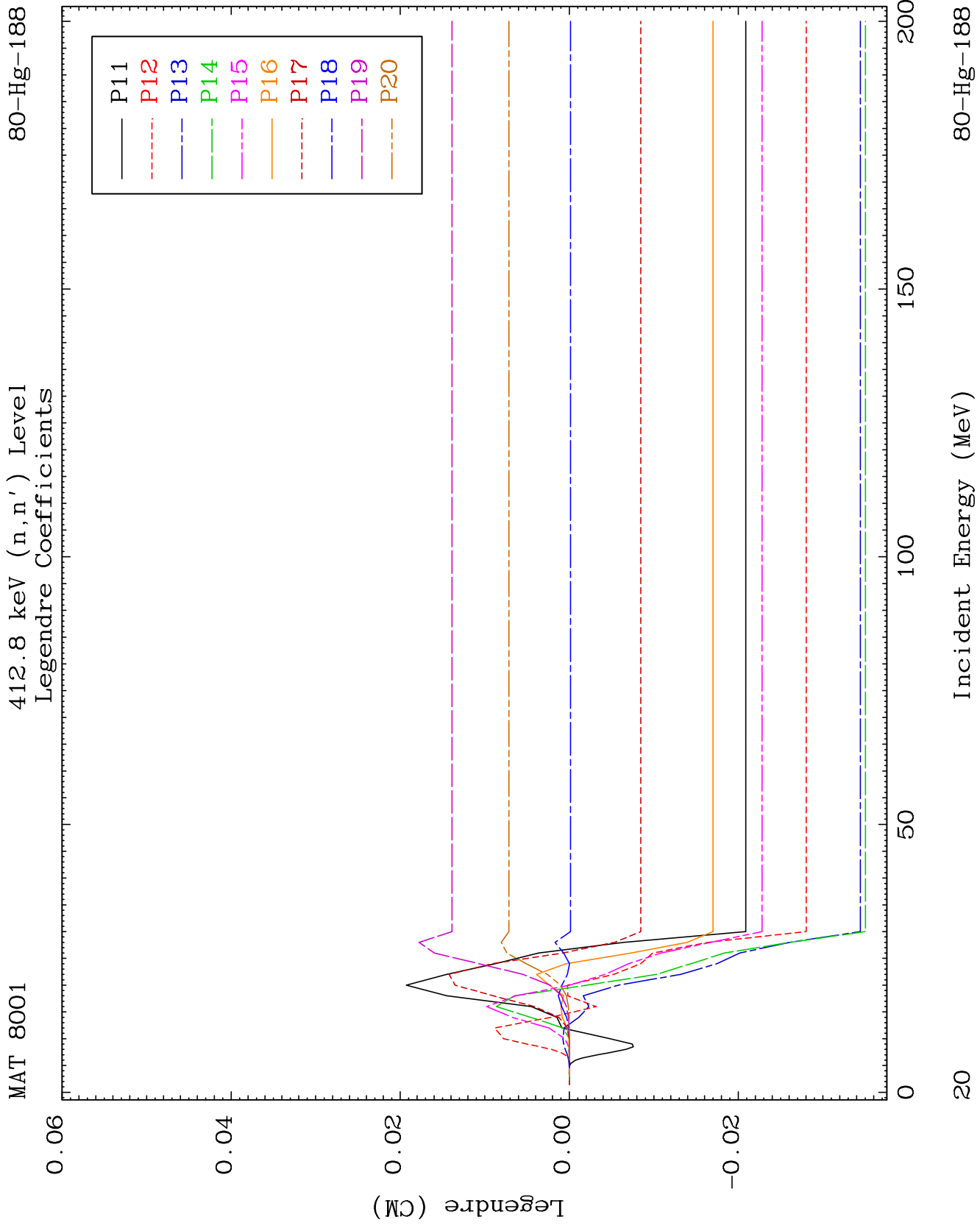


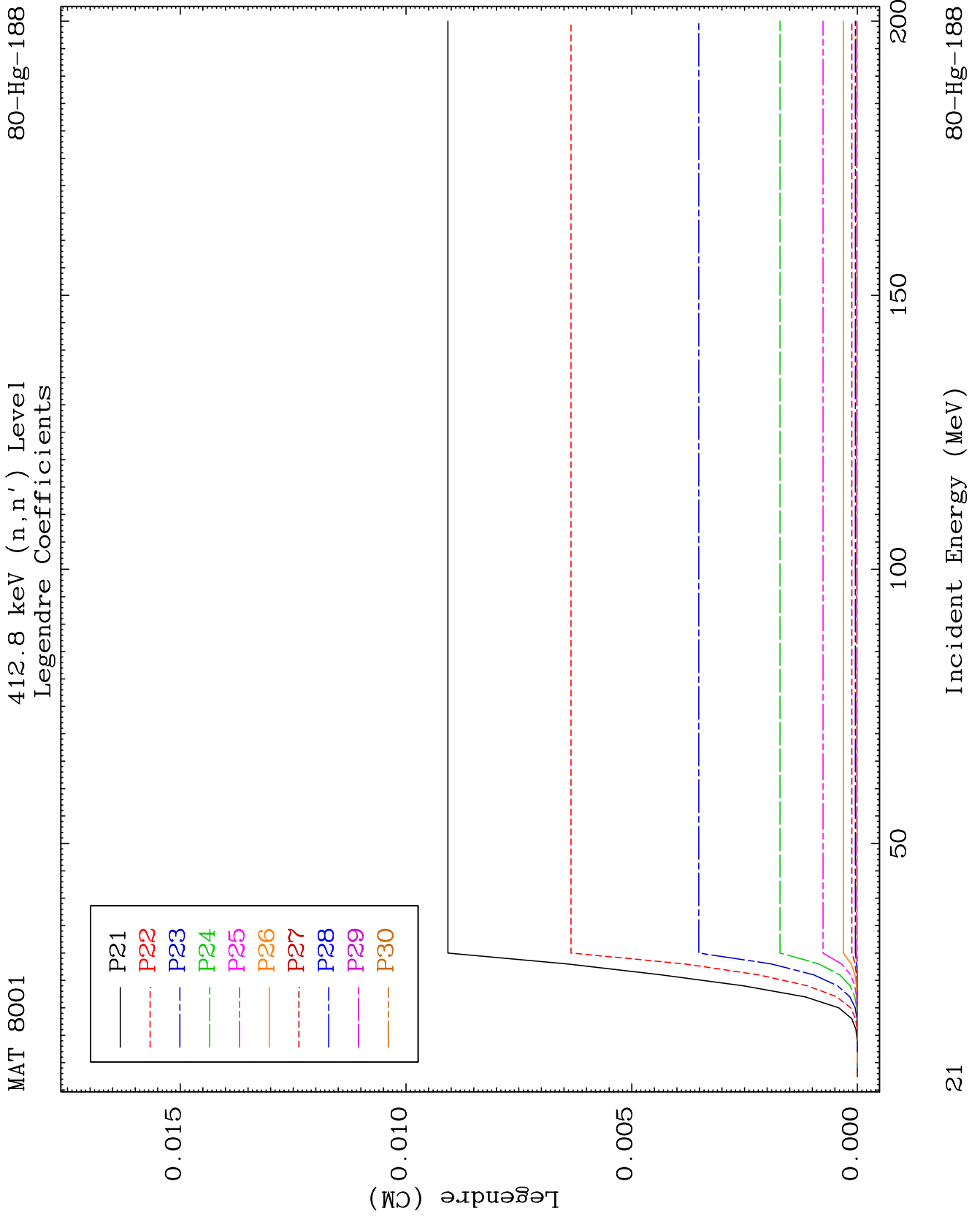


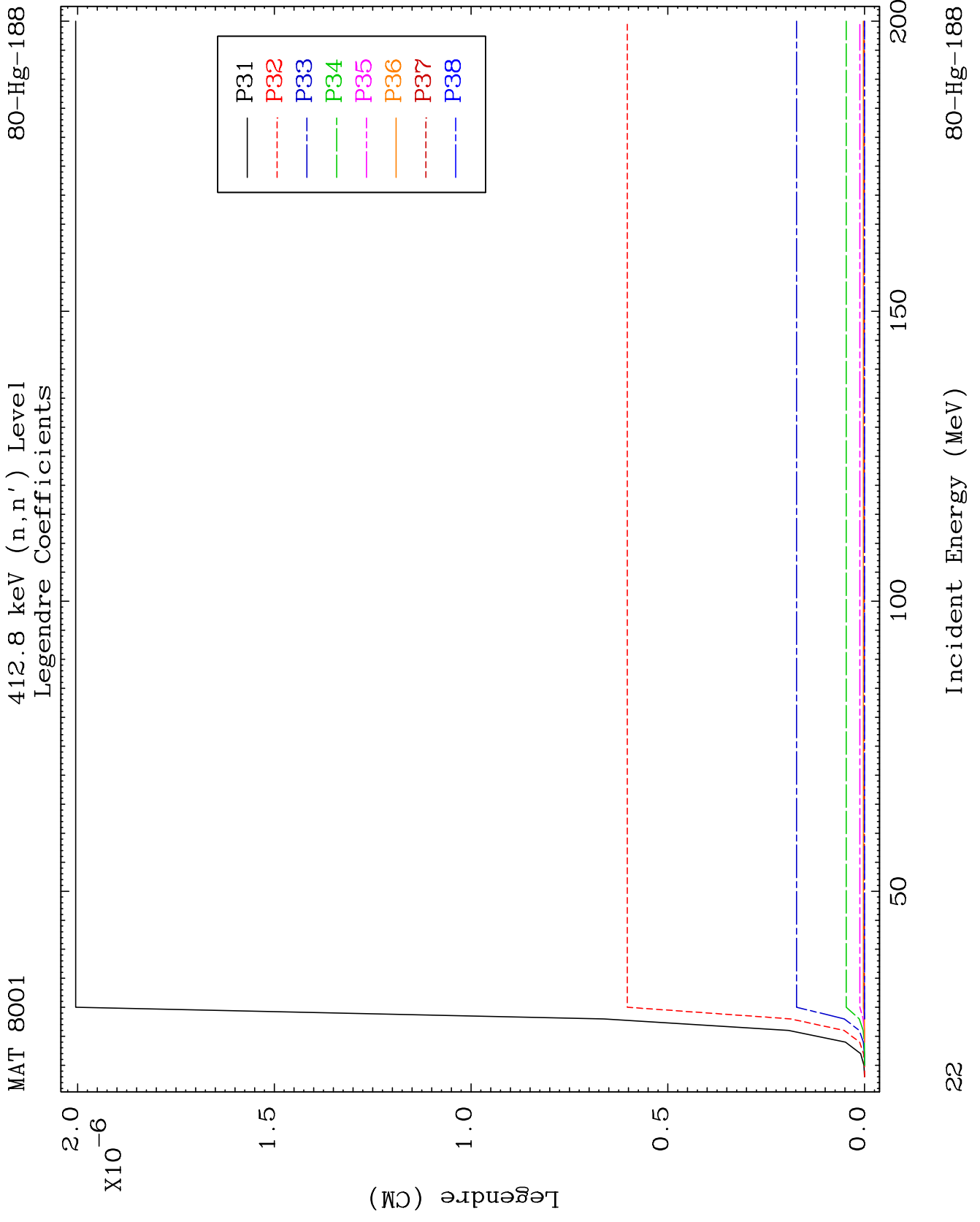


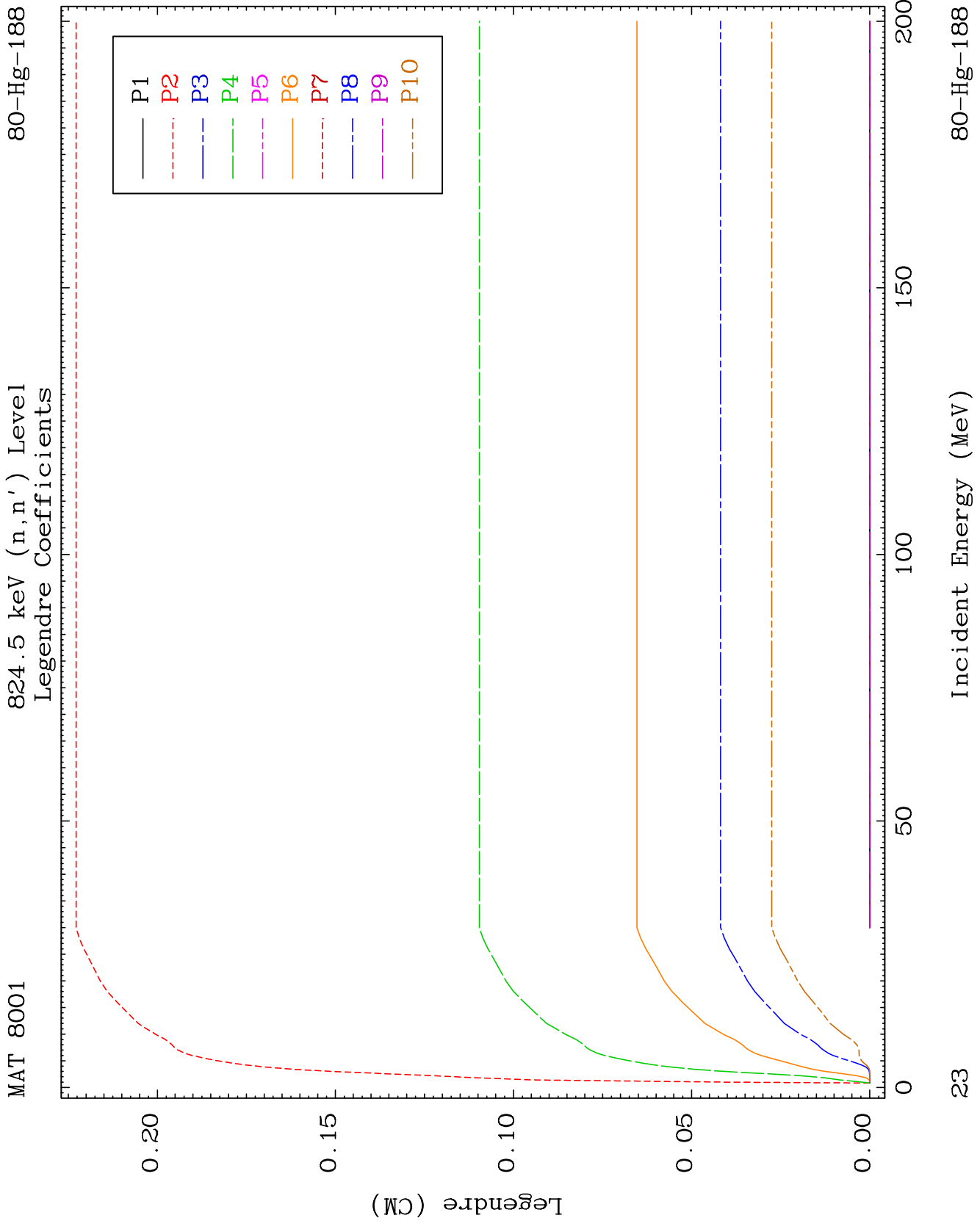




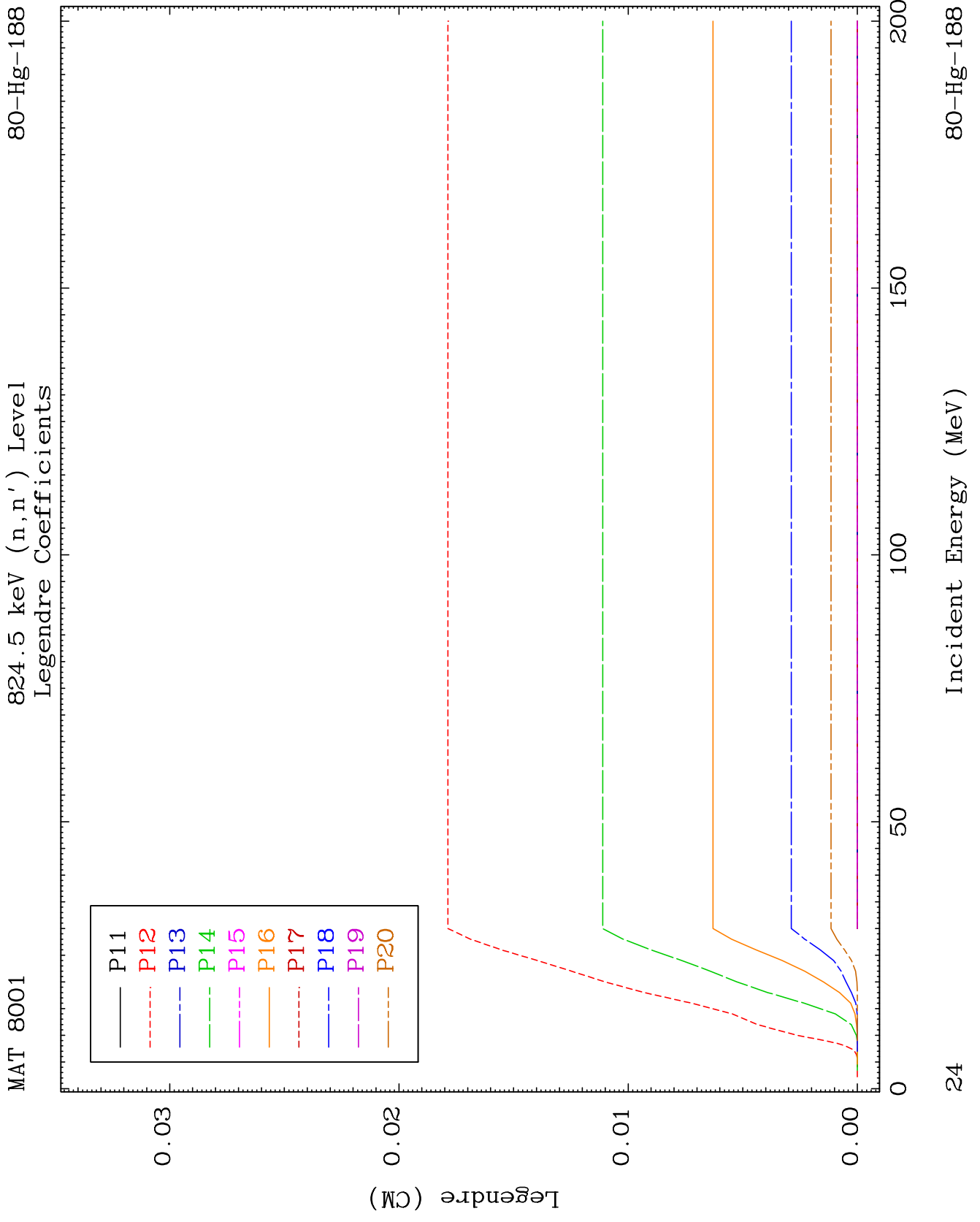


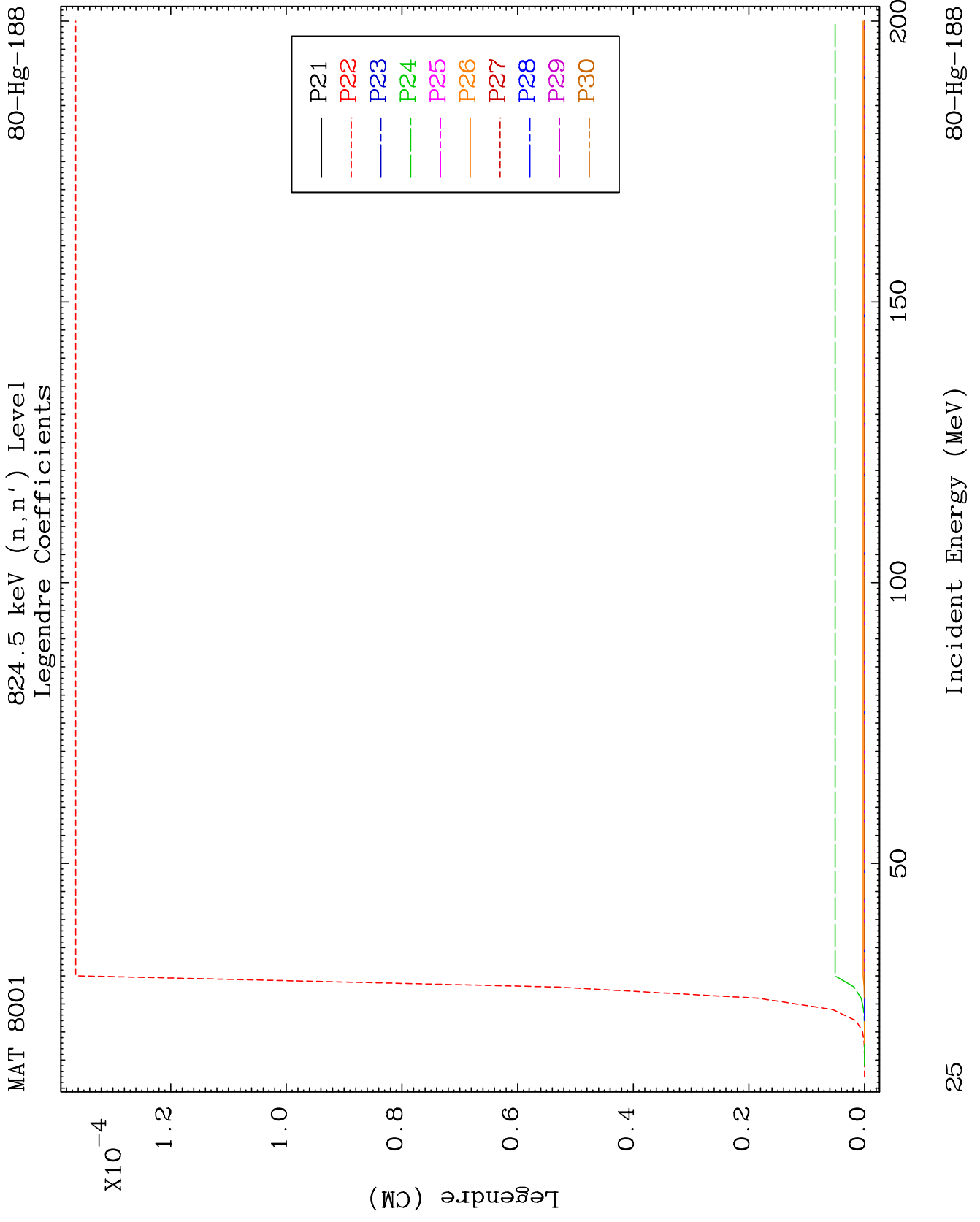


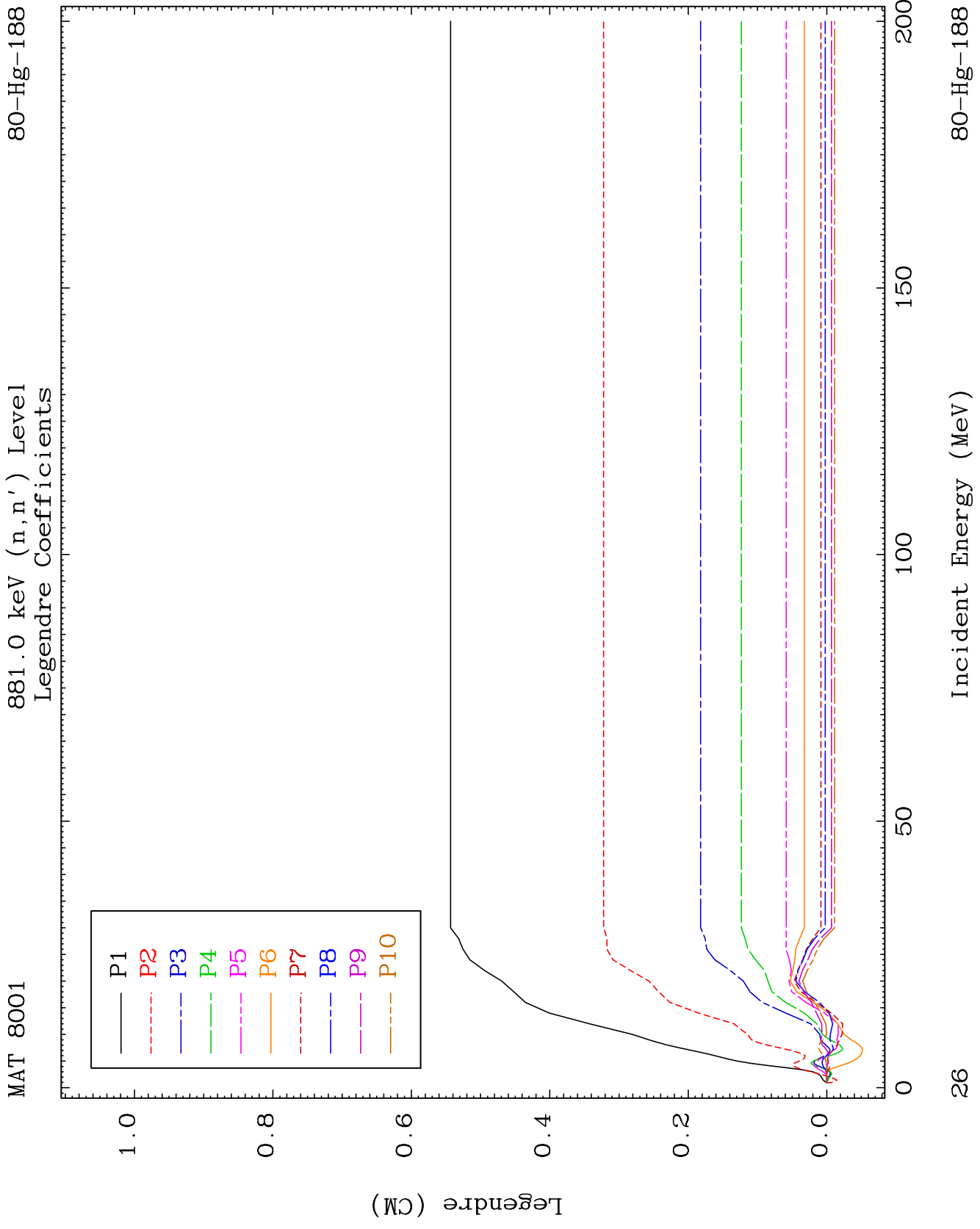










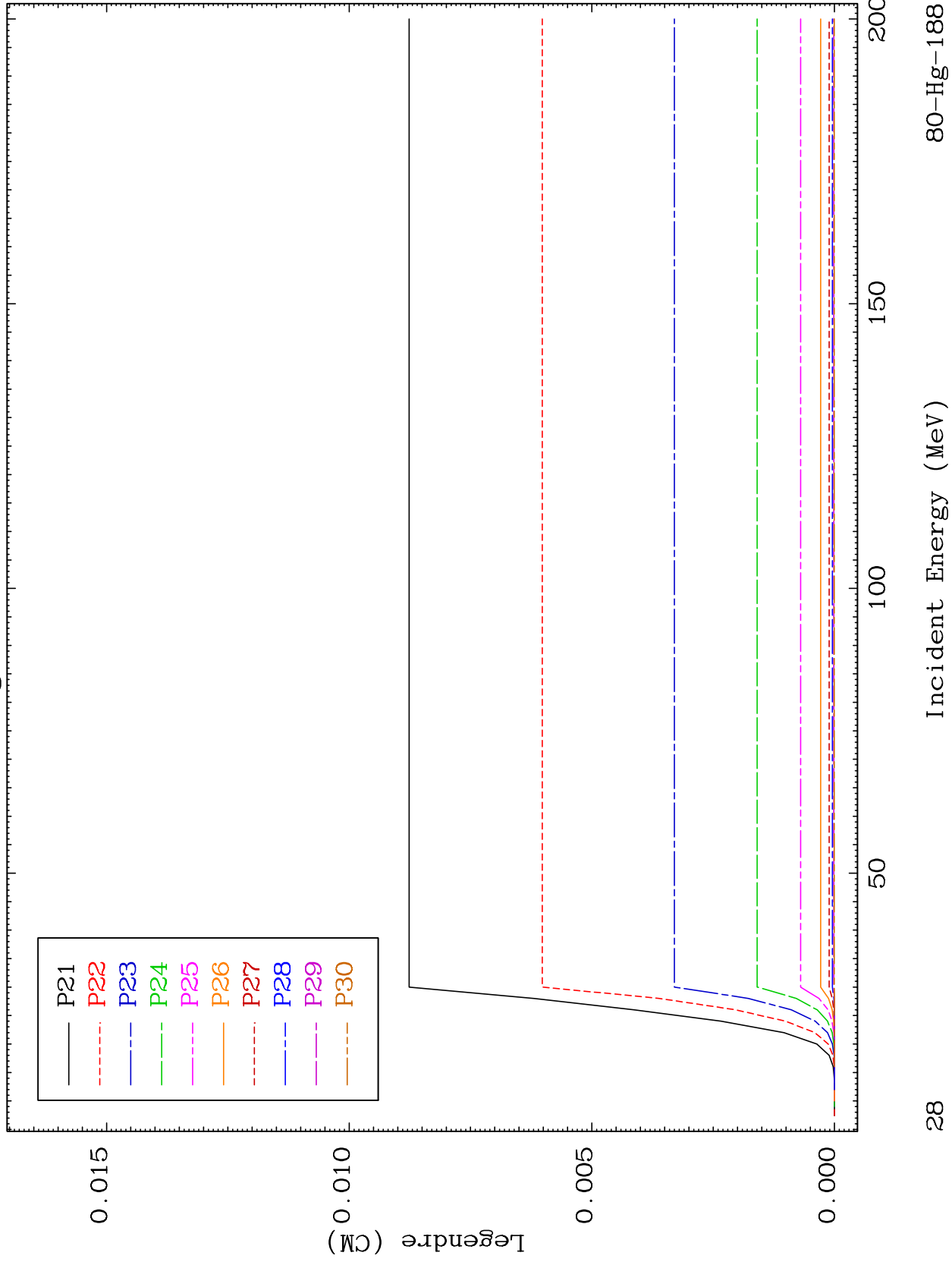




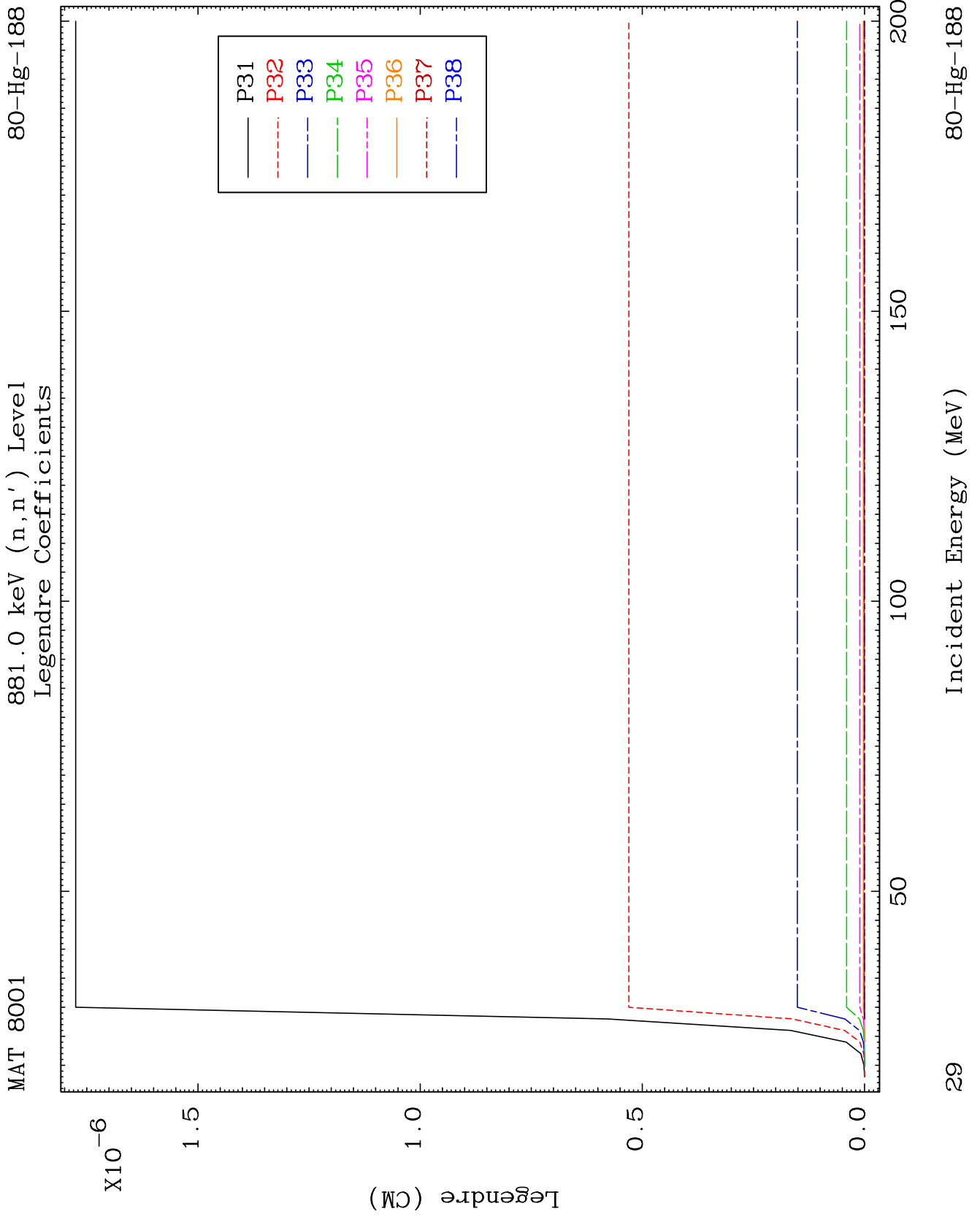
MAT 8001

881.0 keV (n,n') Level  
Legendre Coefficients

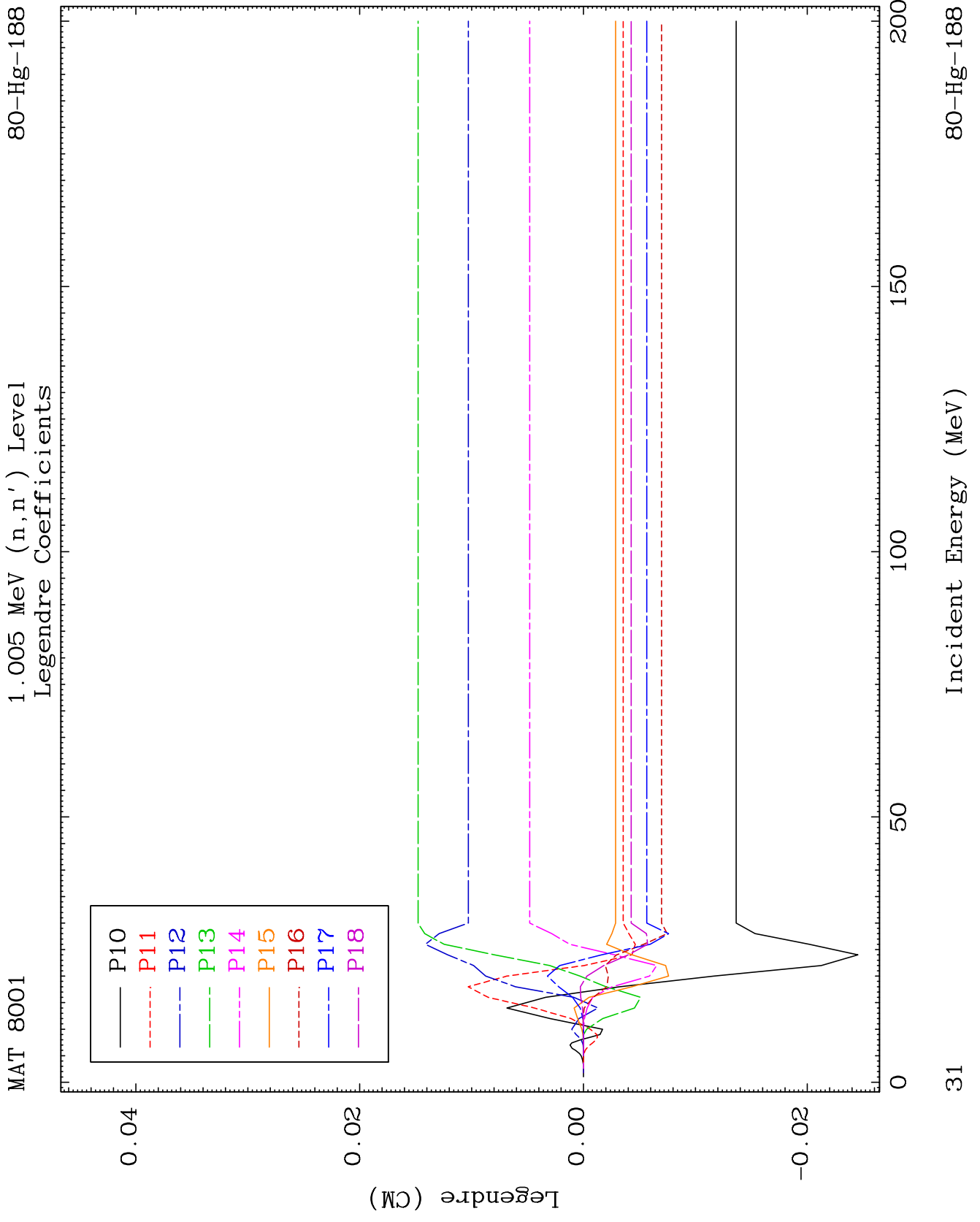
80-Hg-188



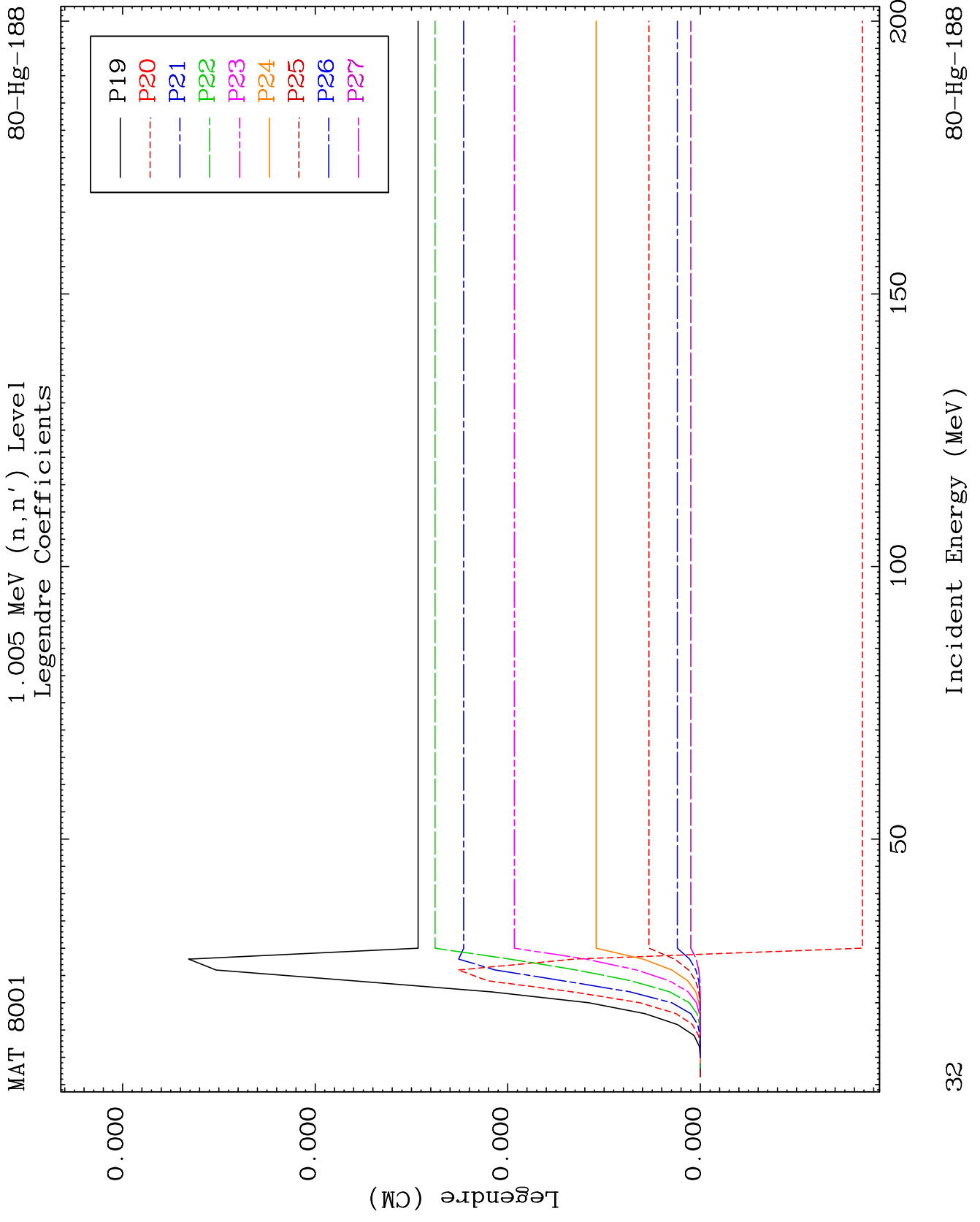
28

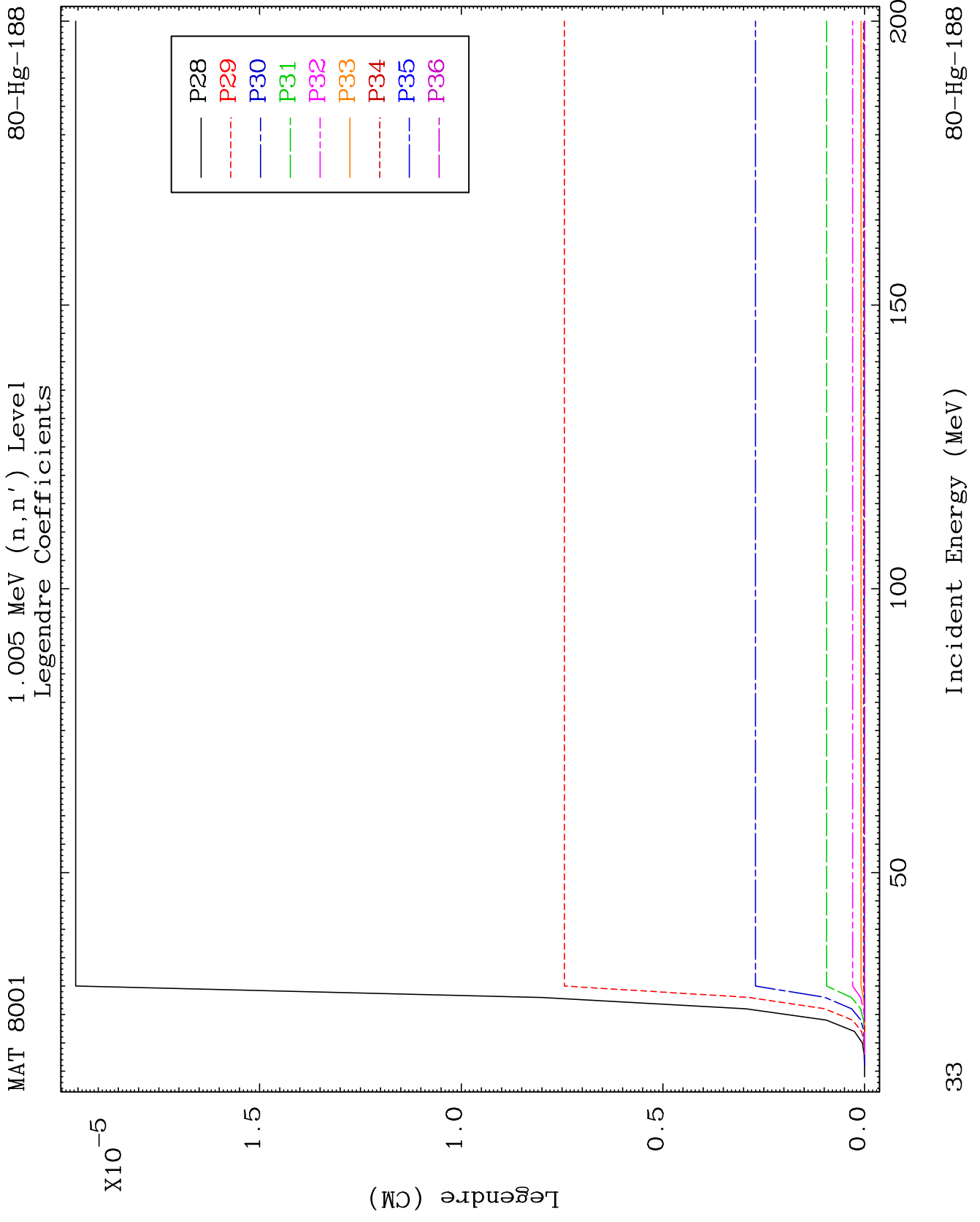


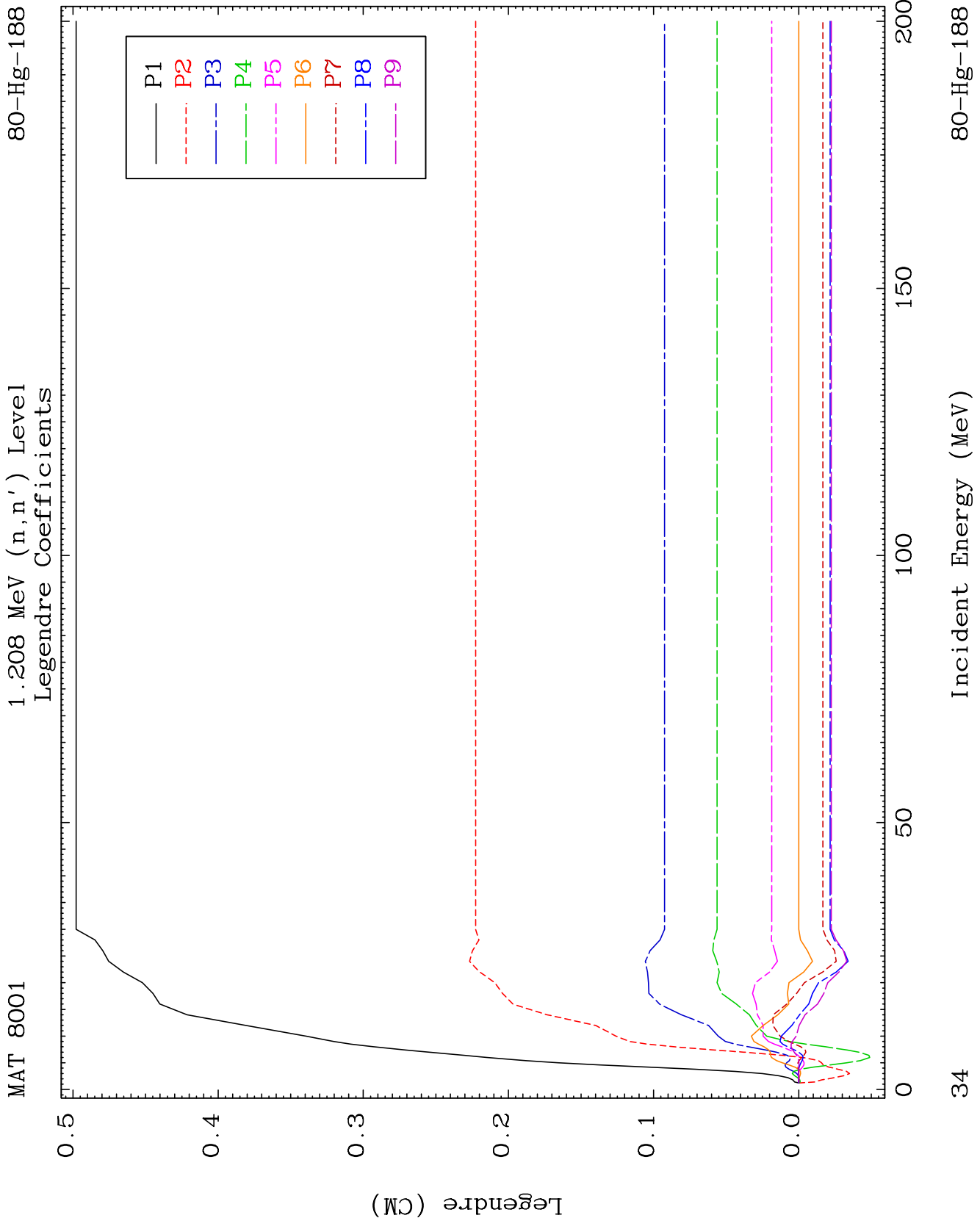


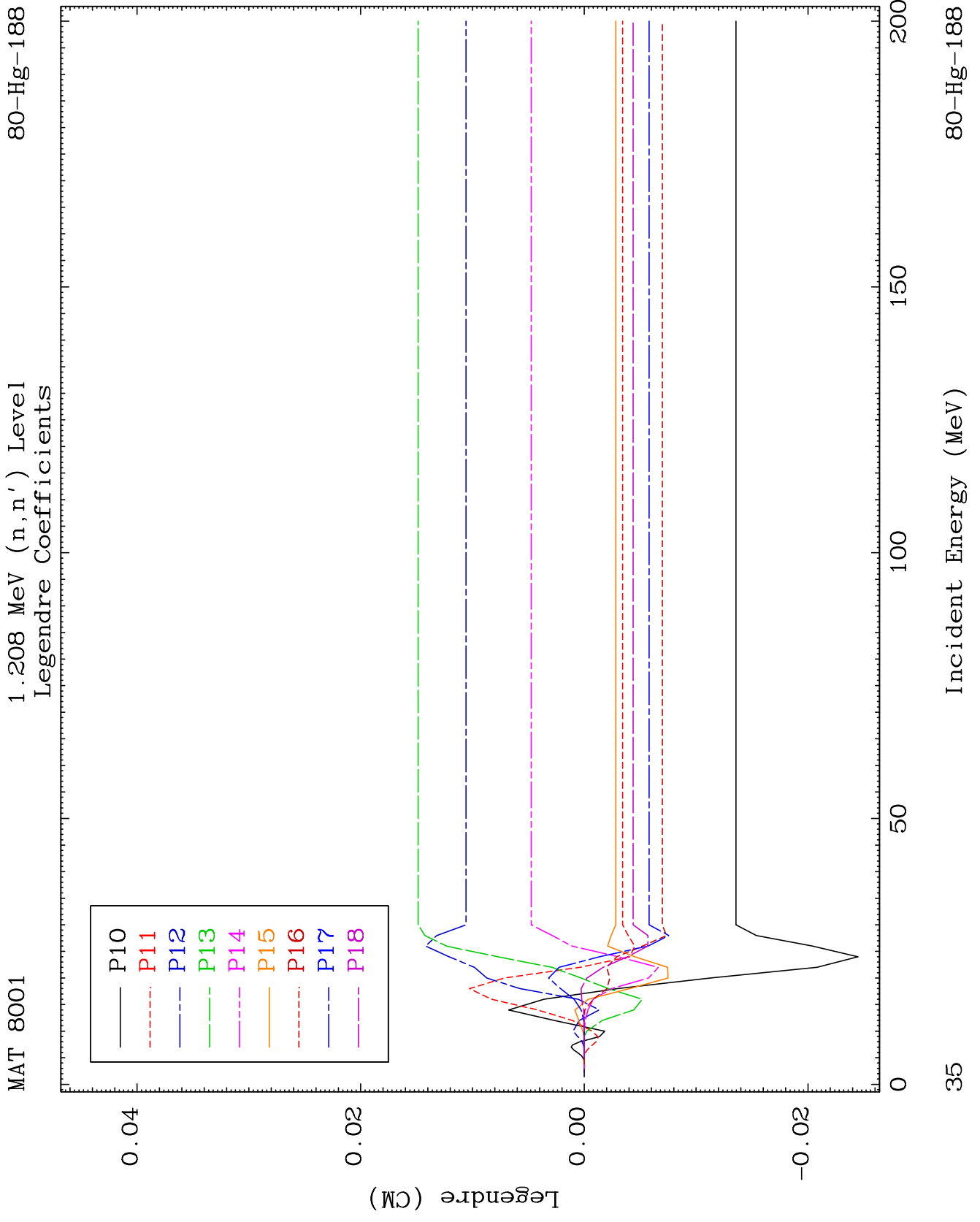


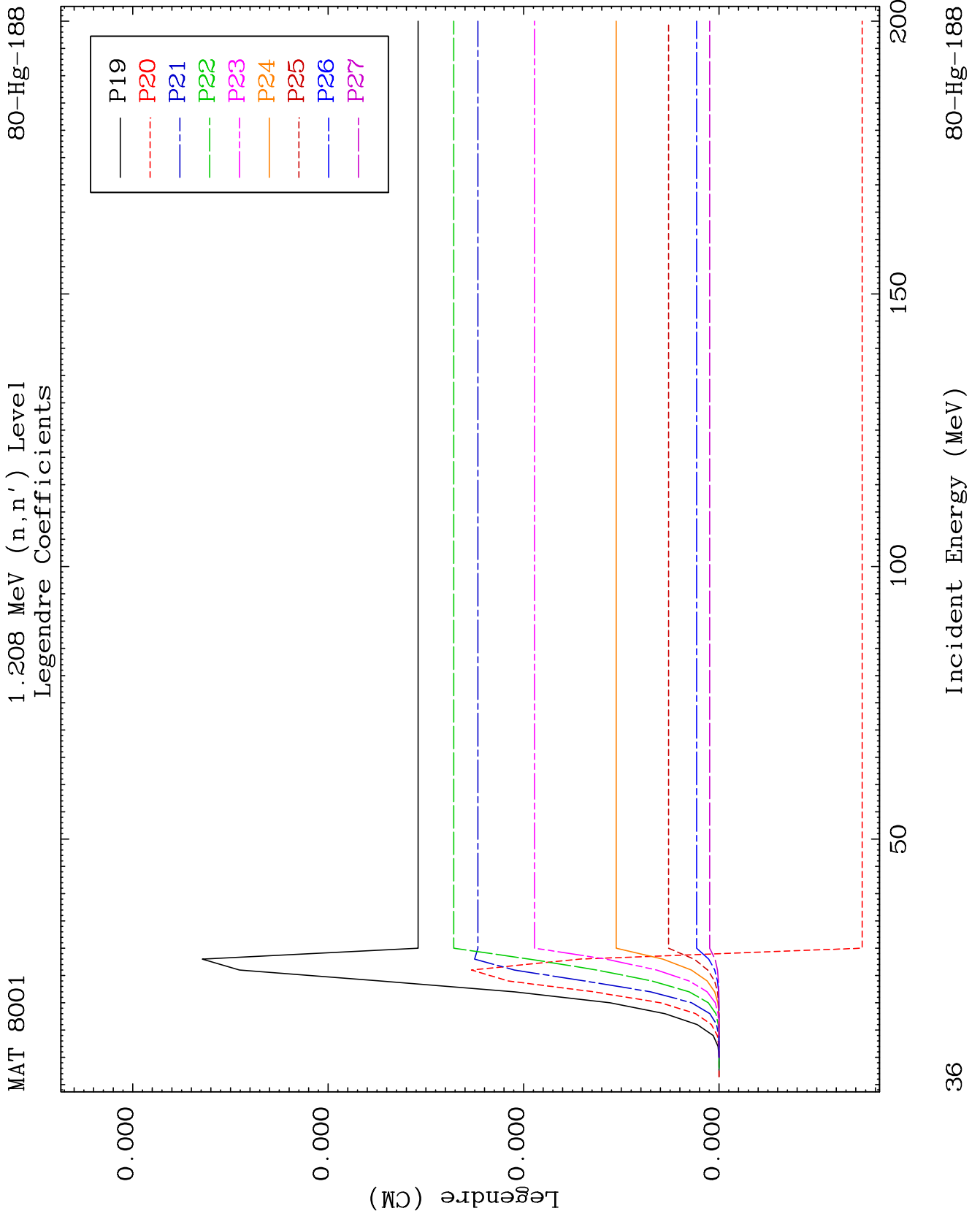


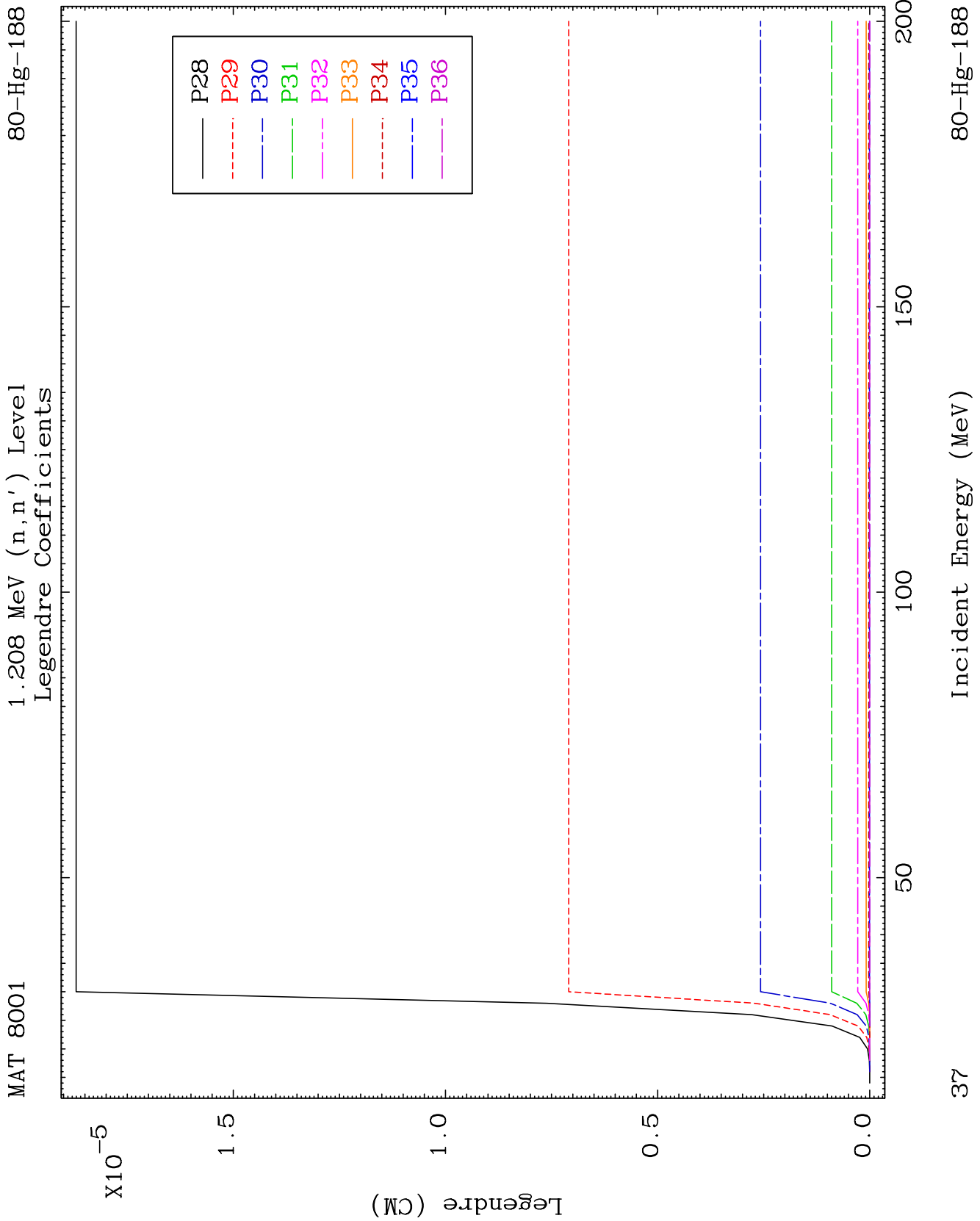


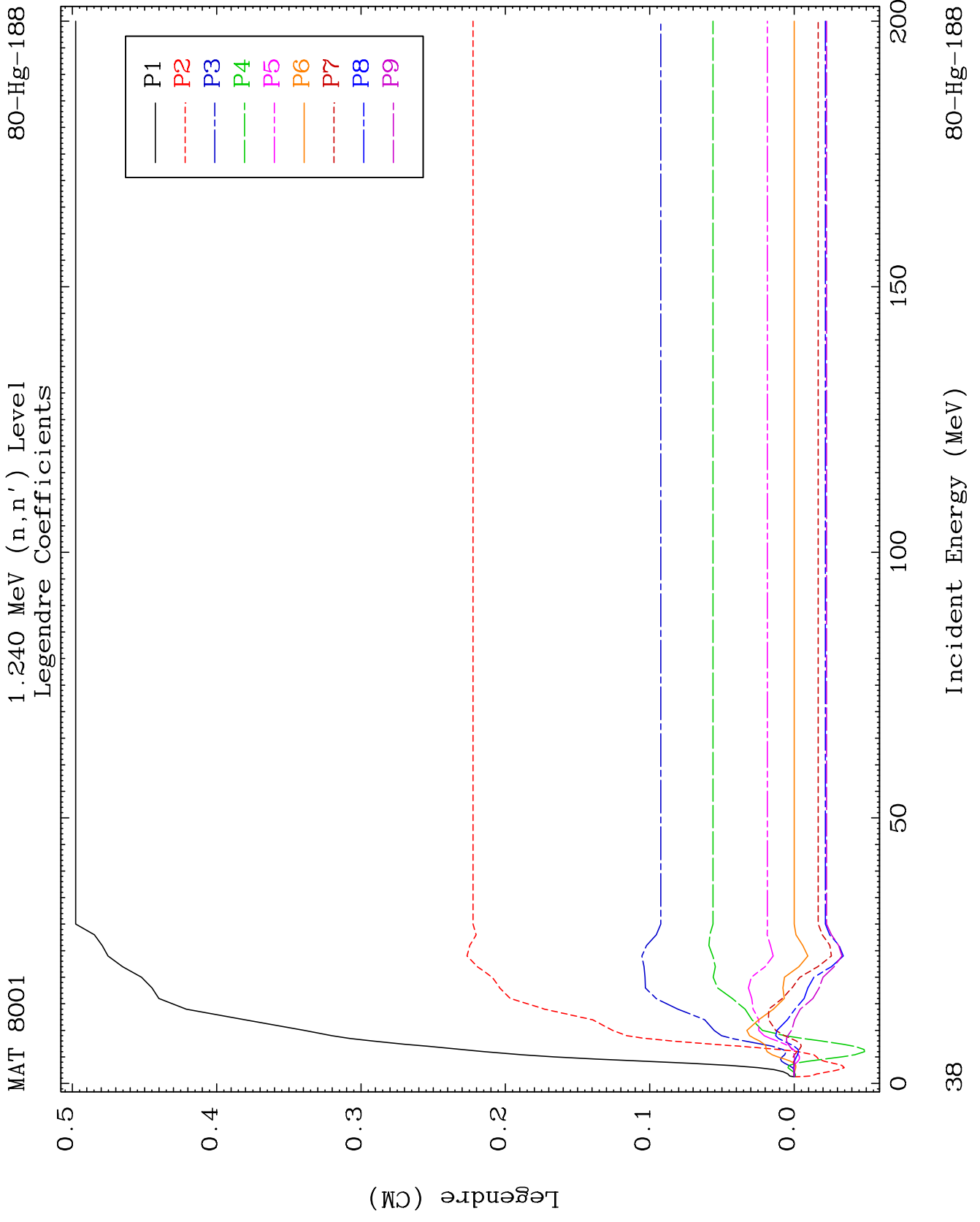






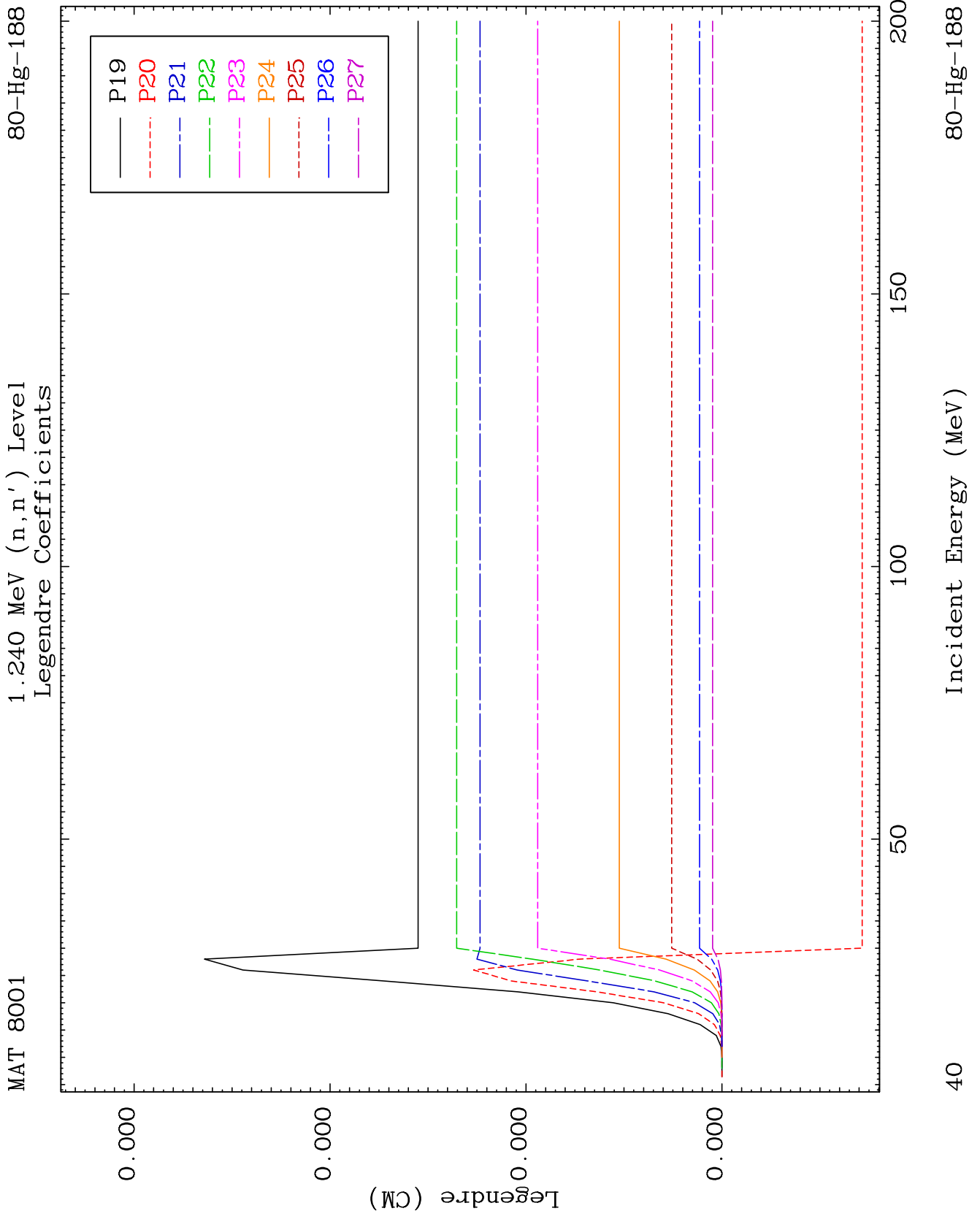


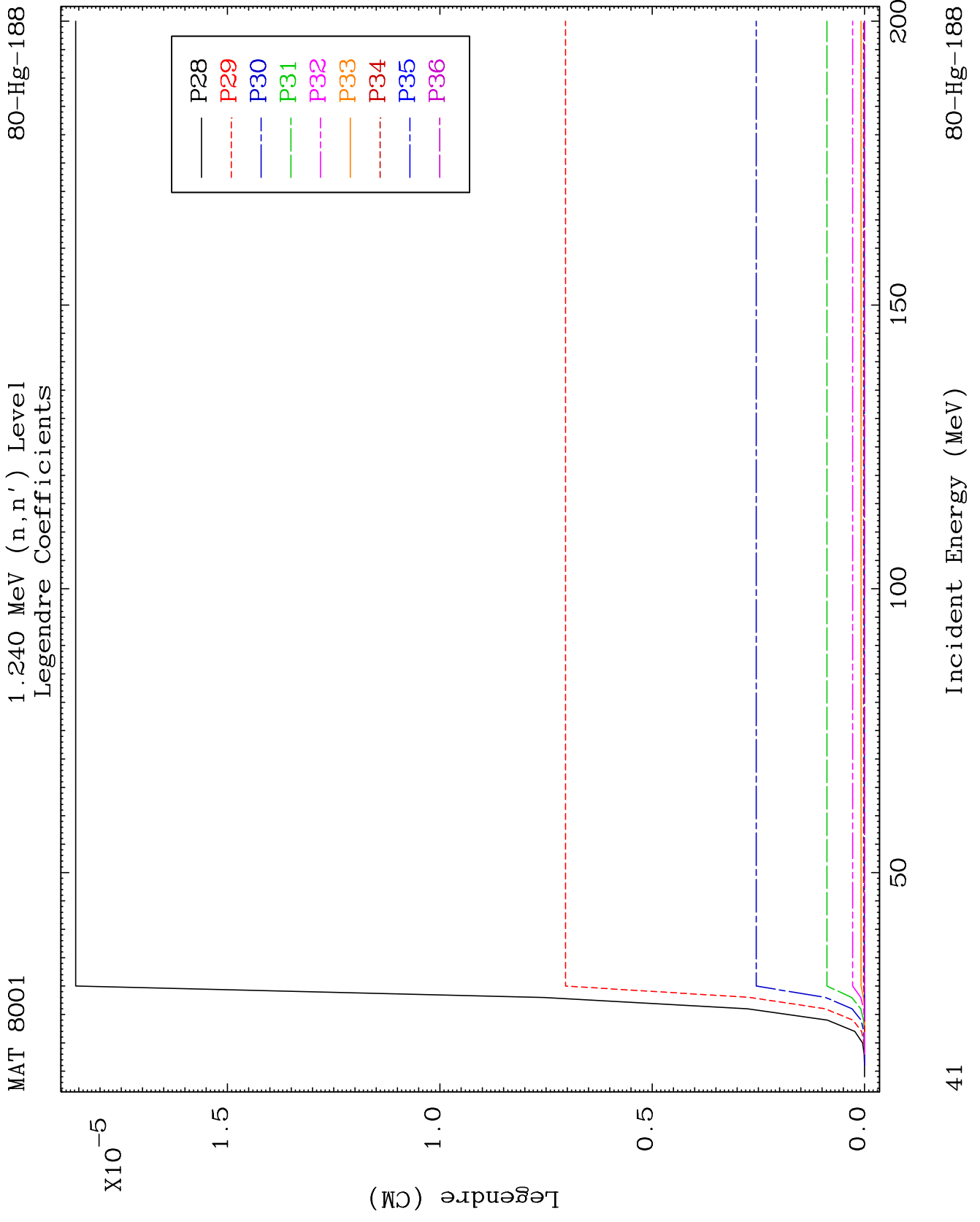


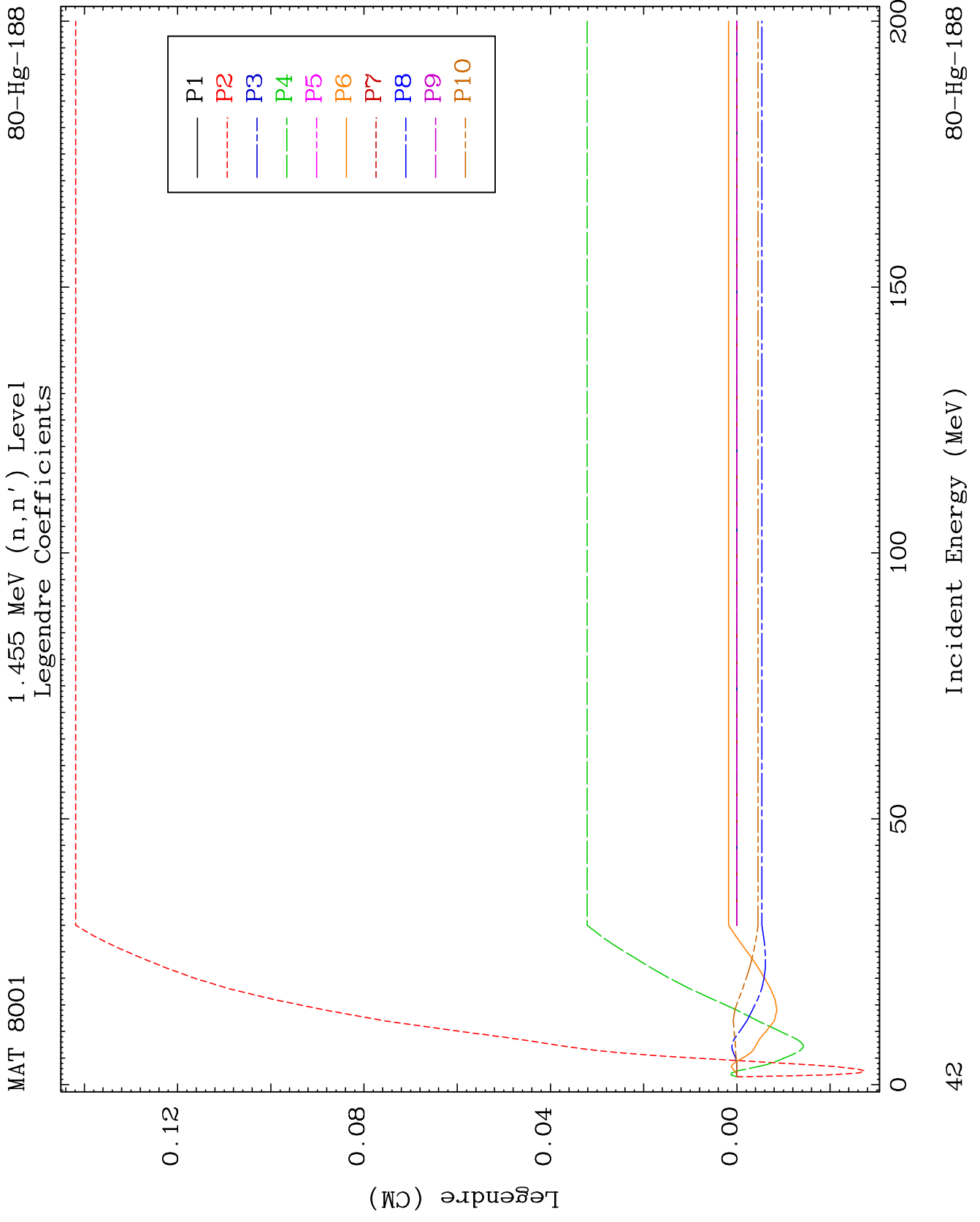


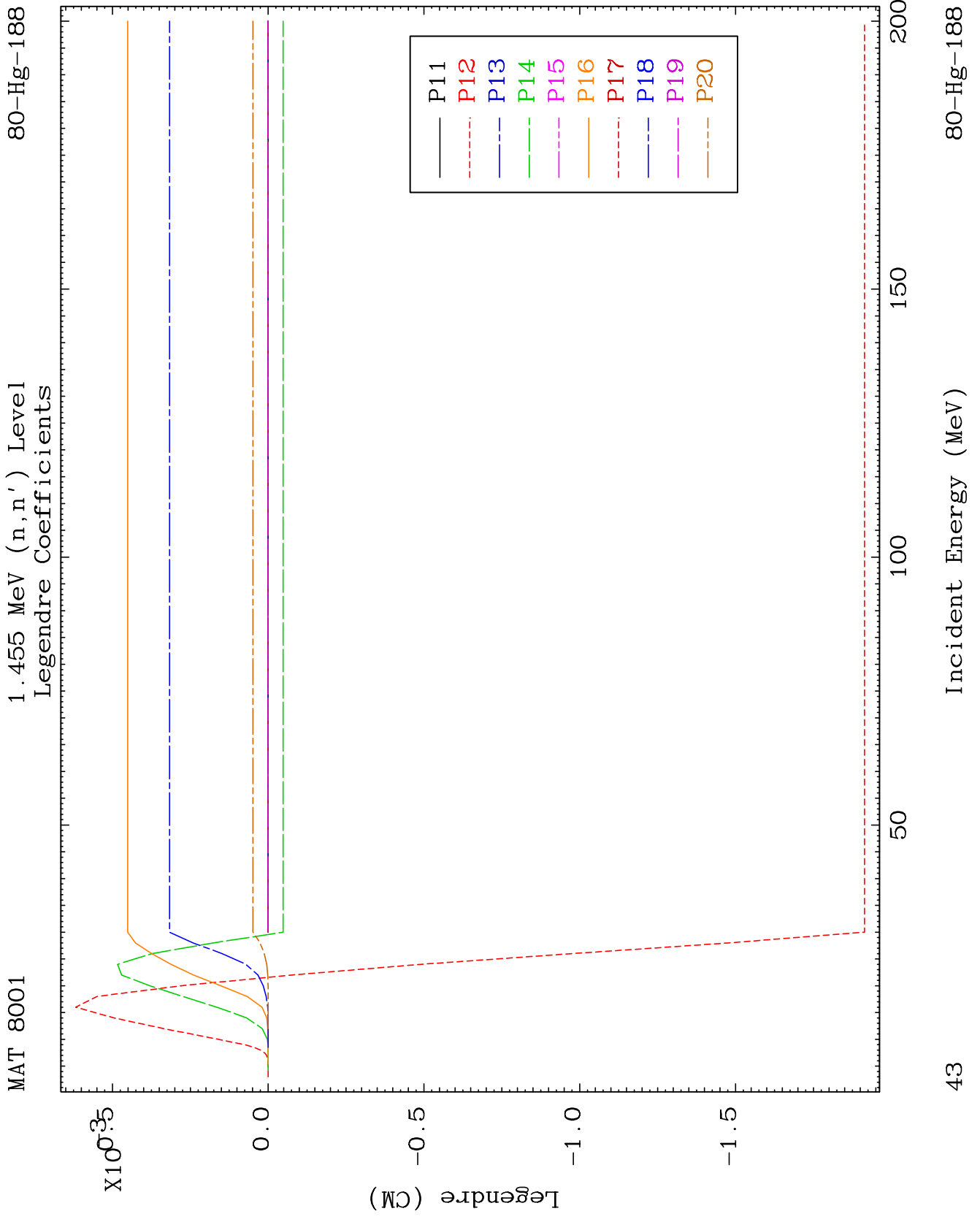


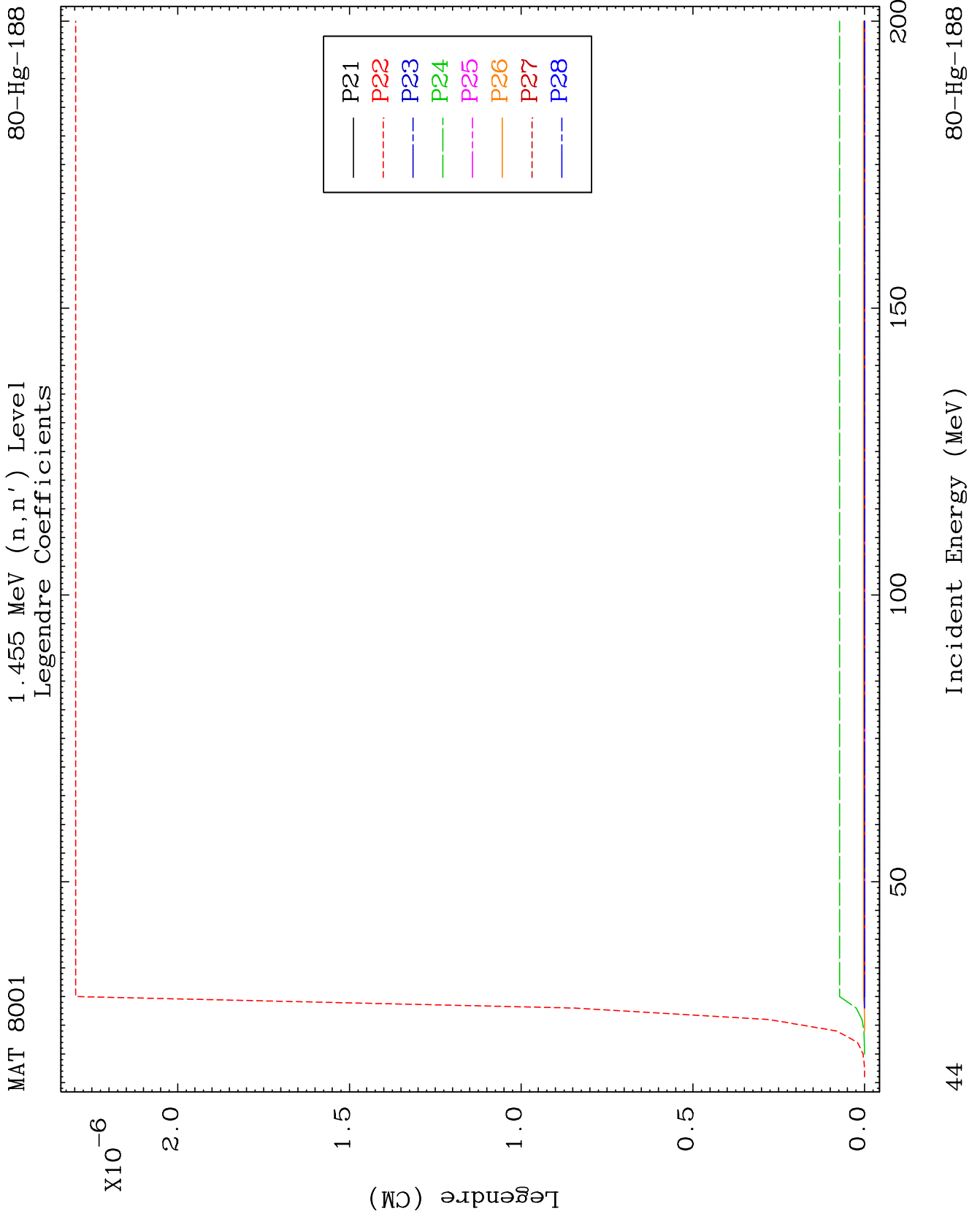






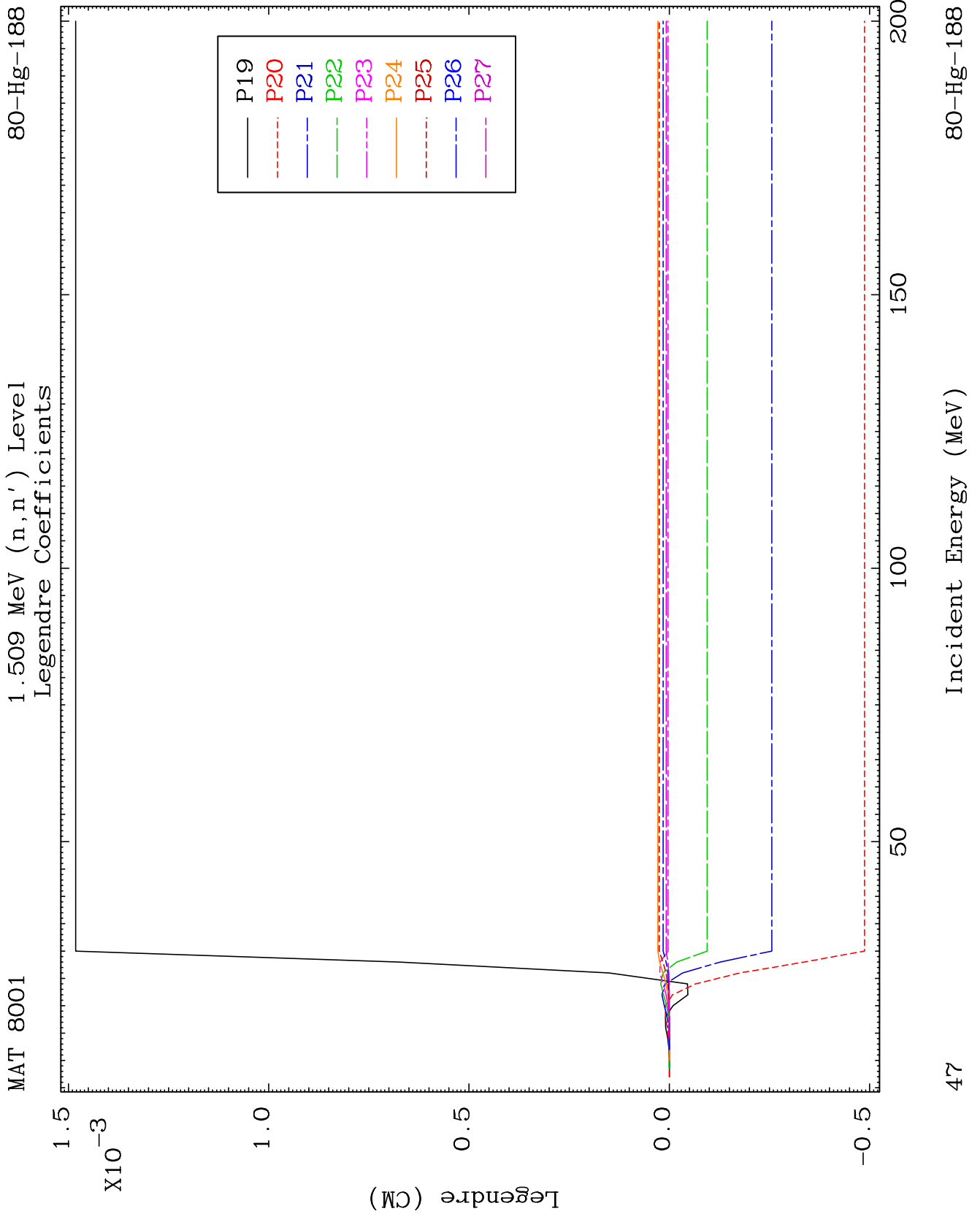




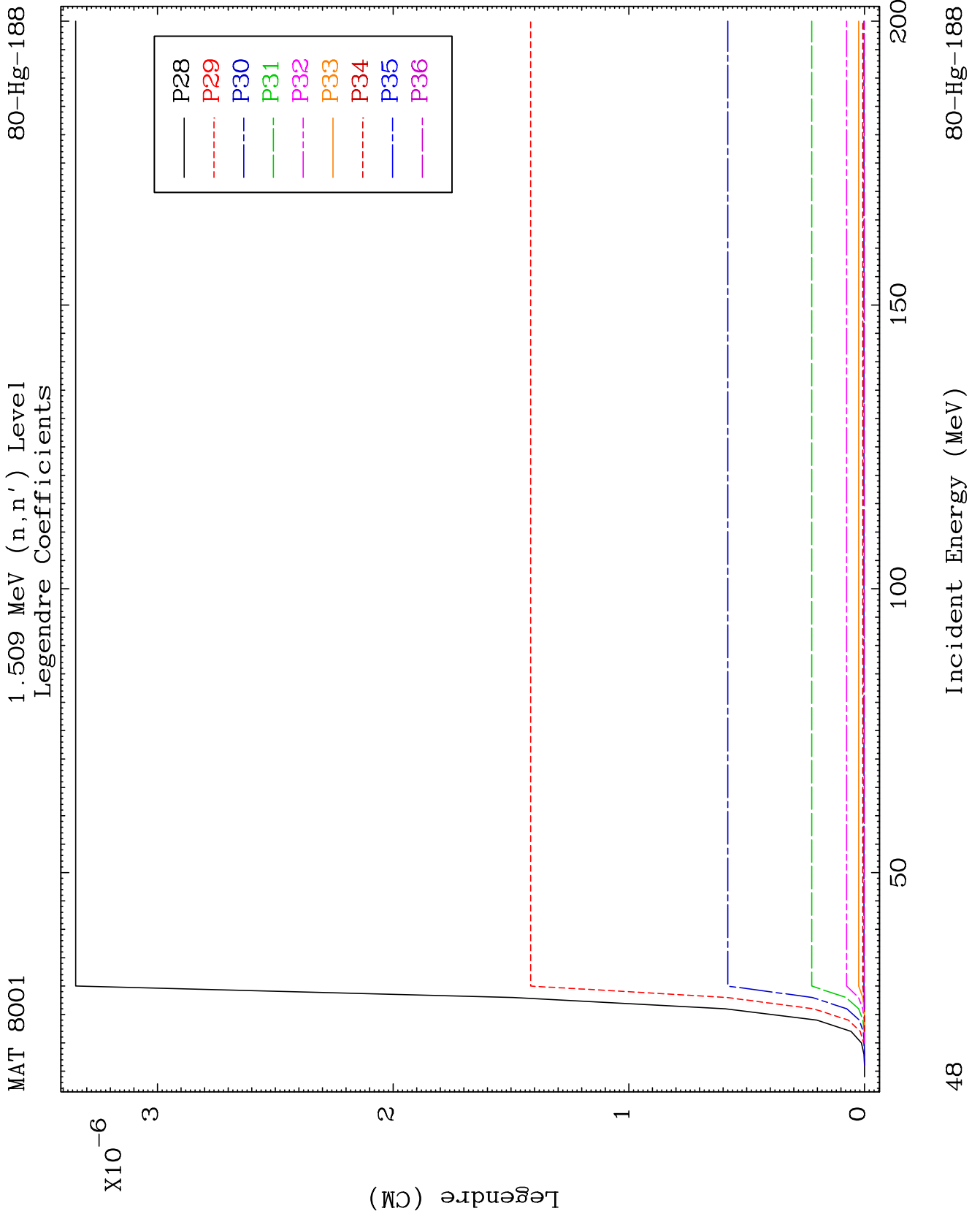


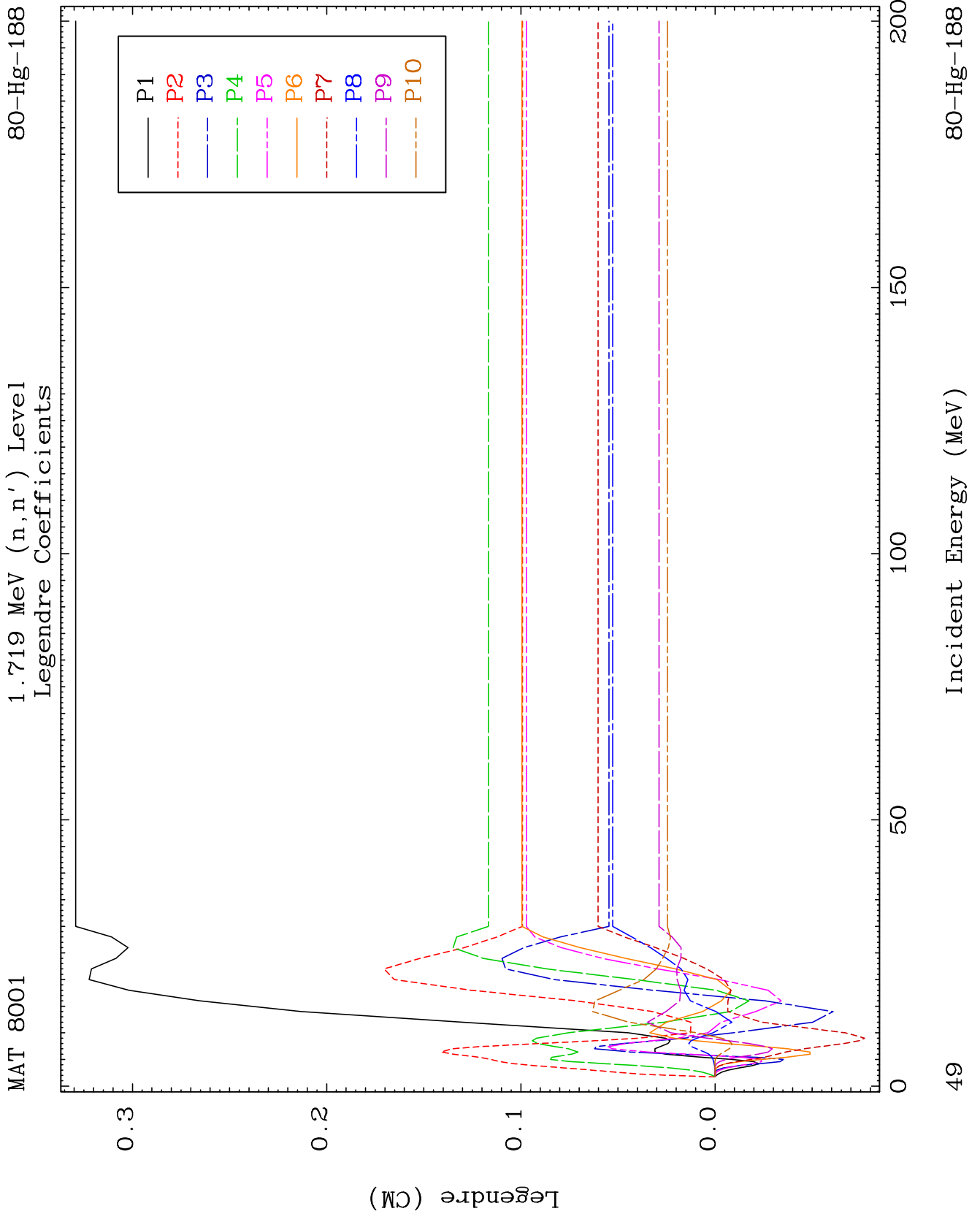




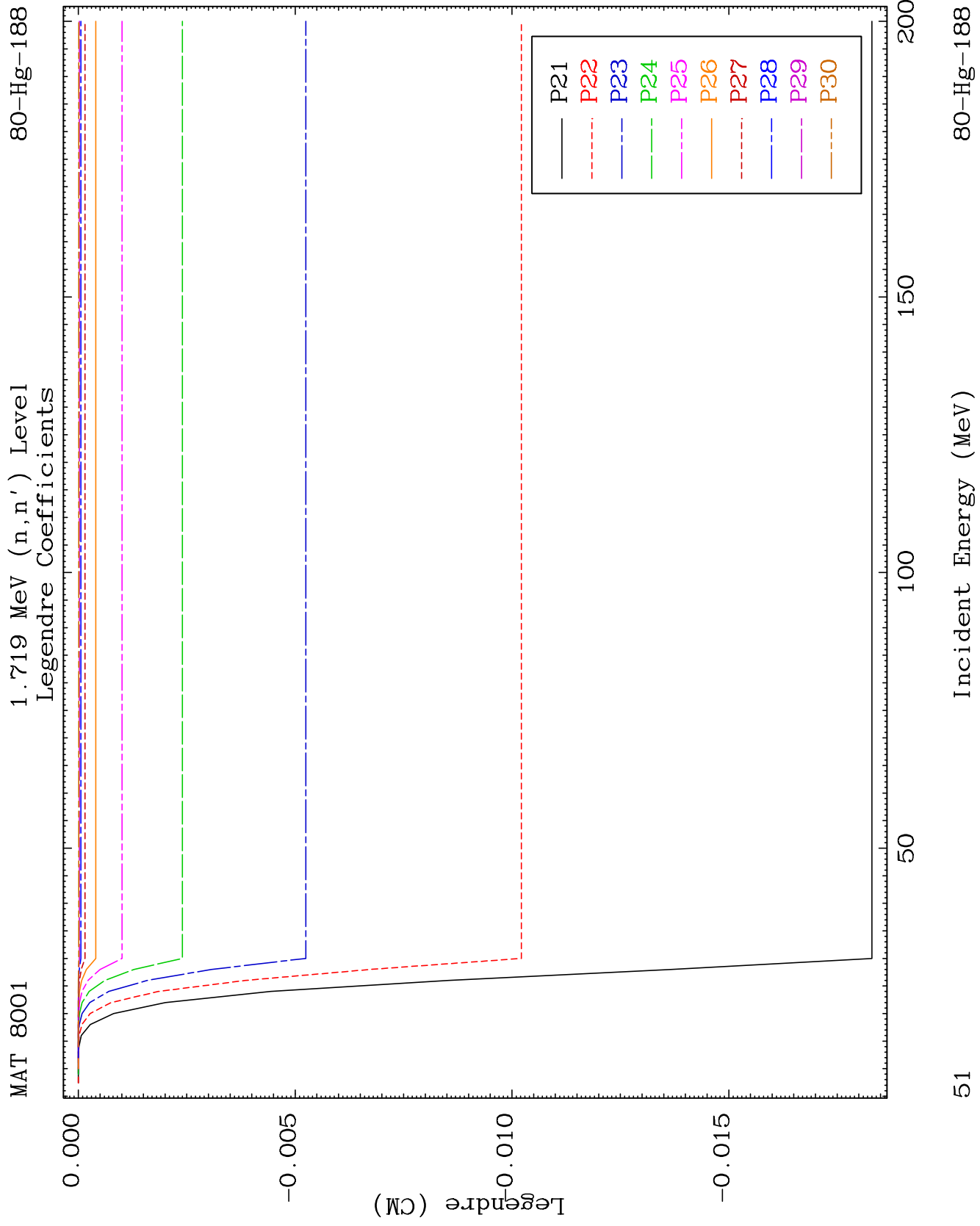


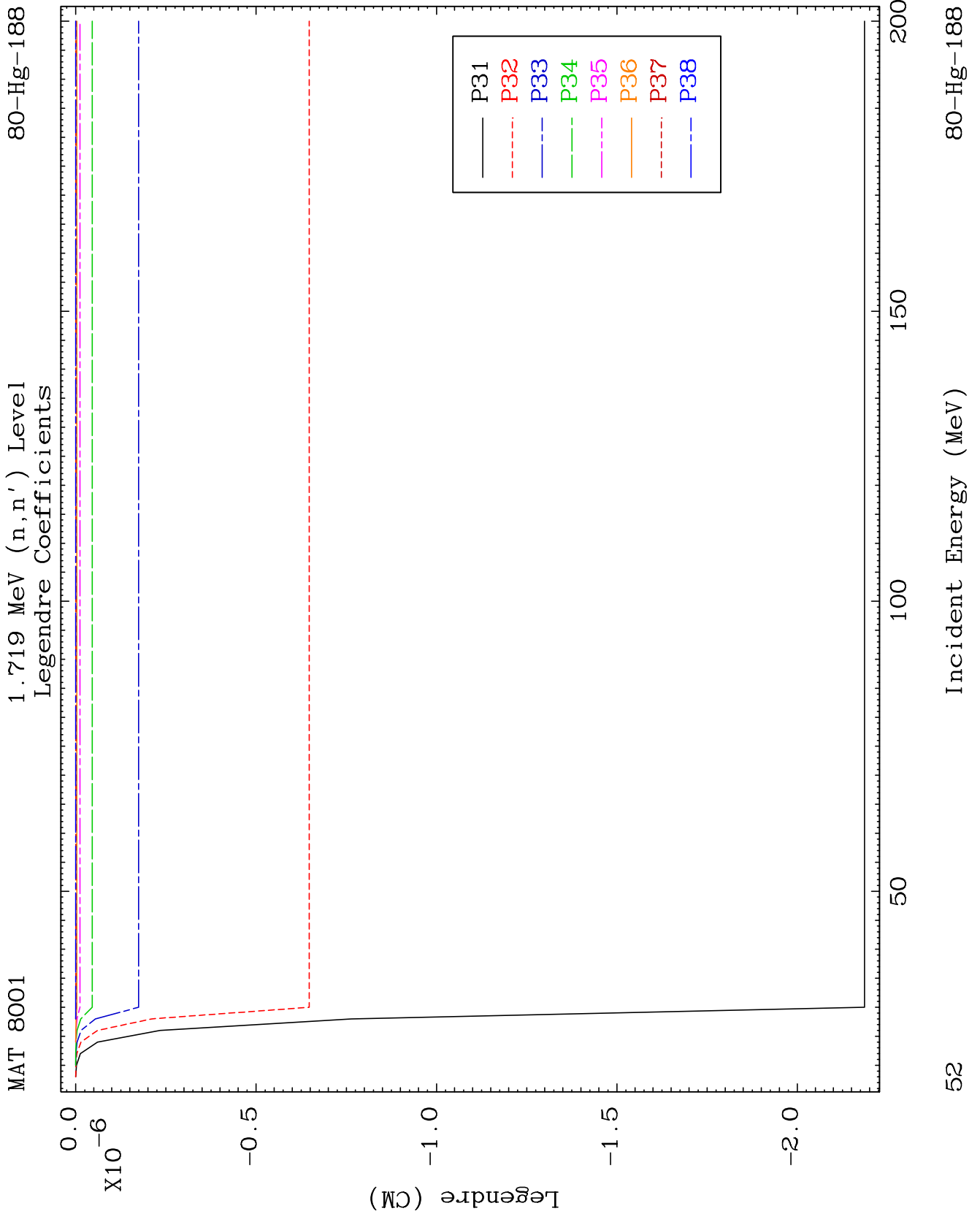


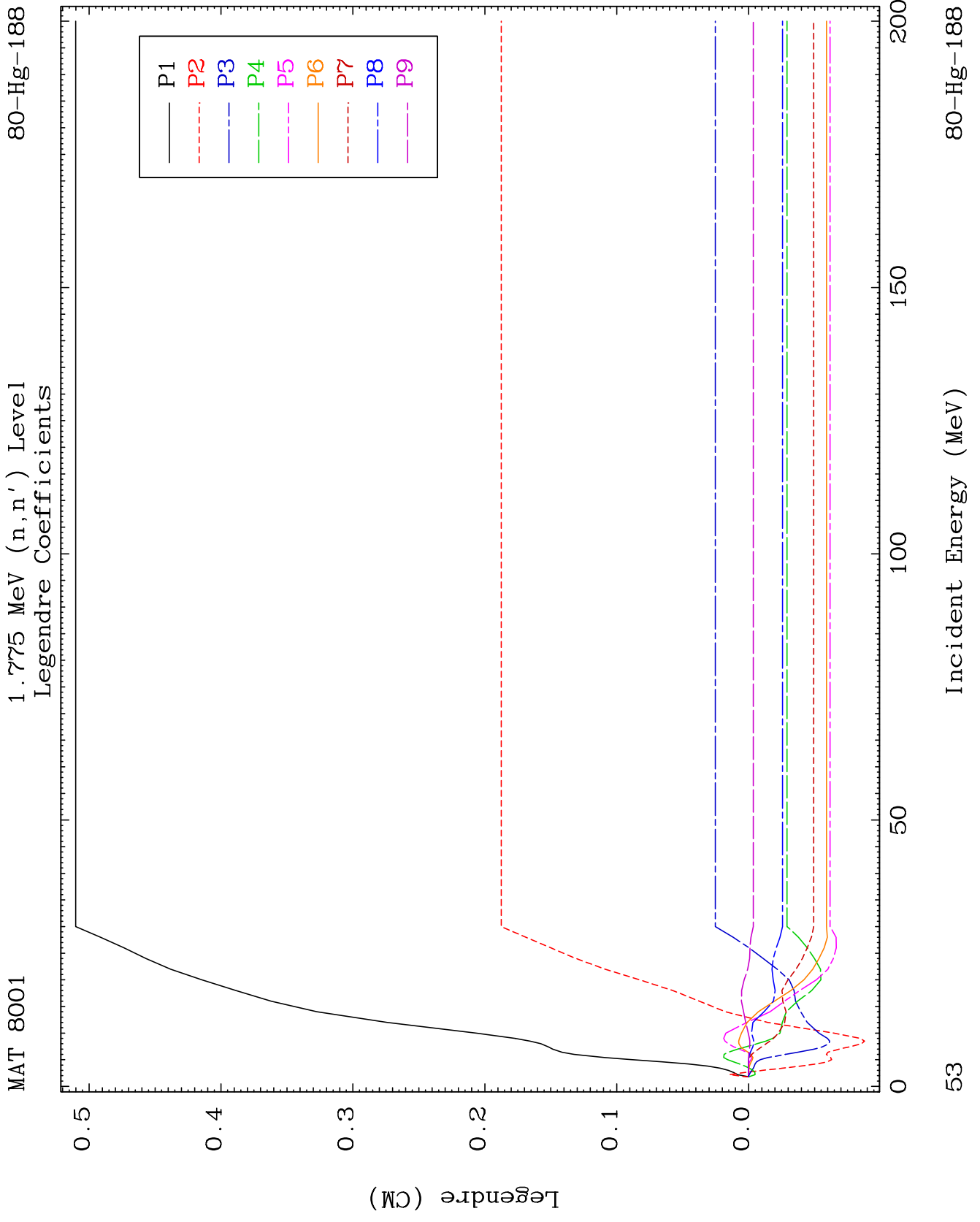




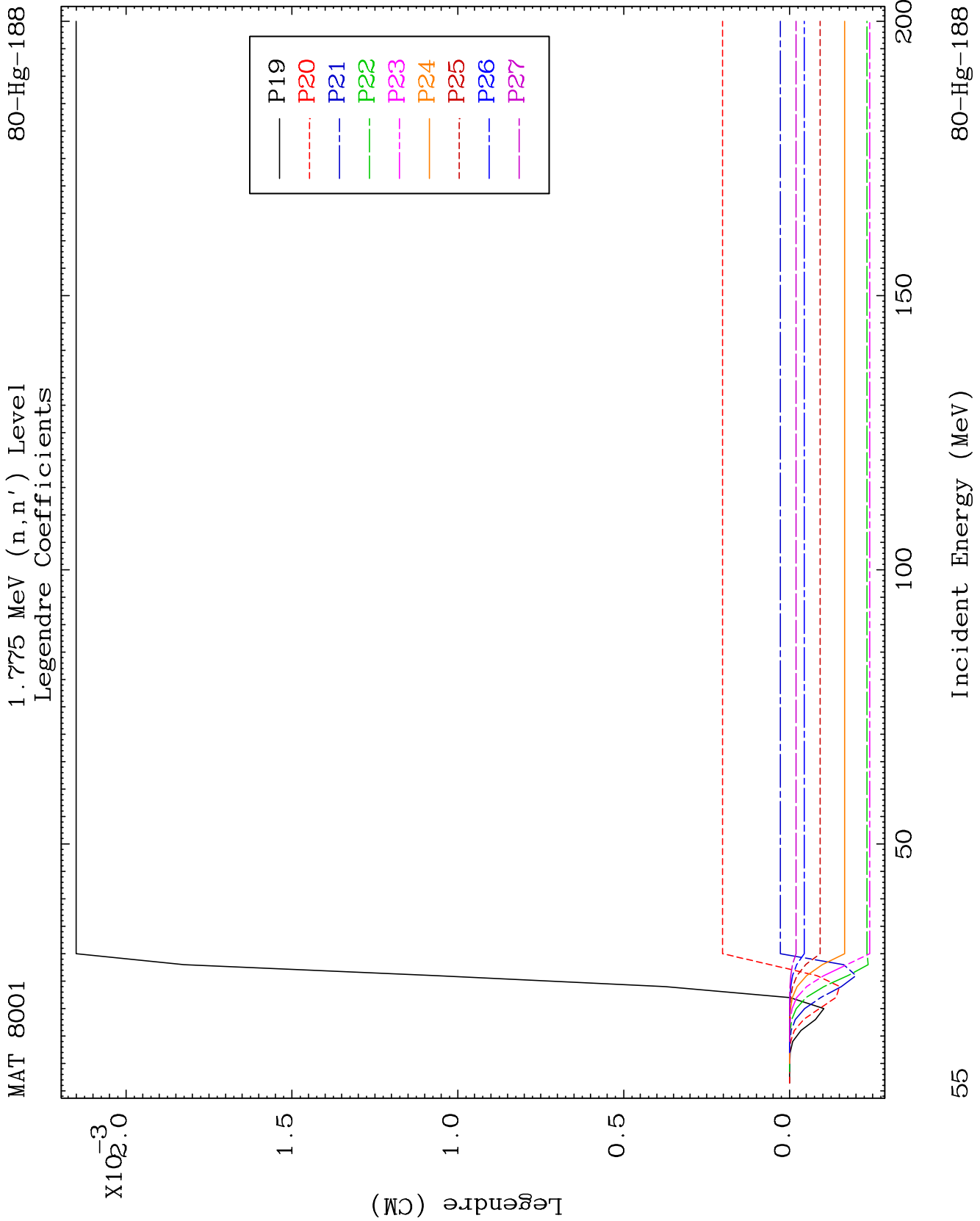




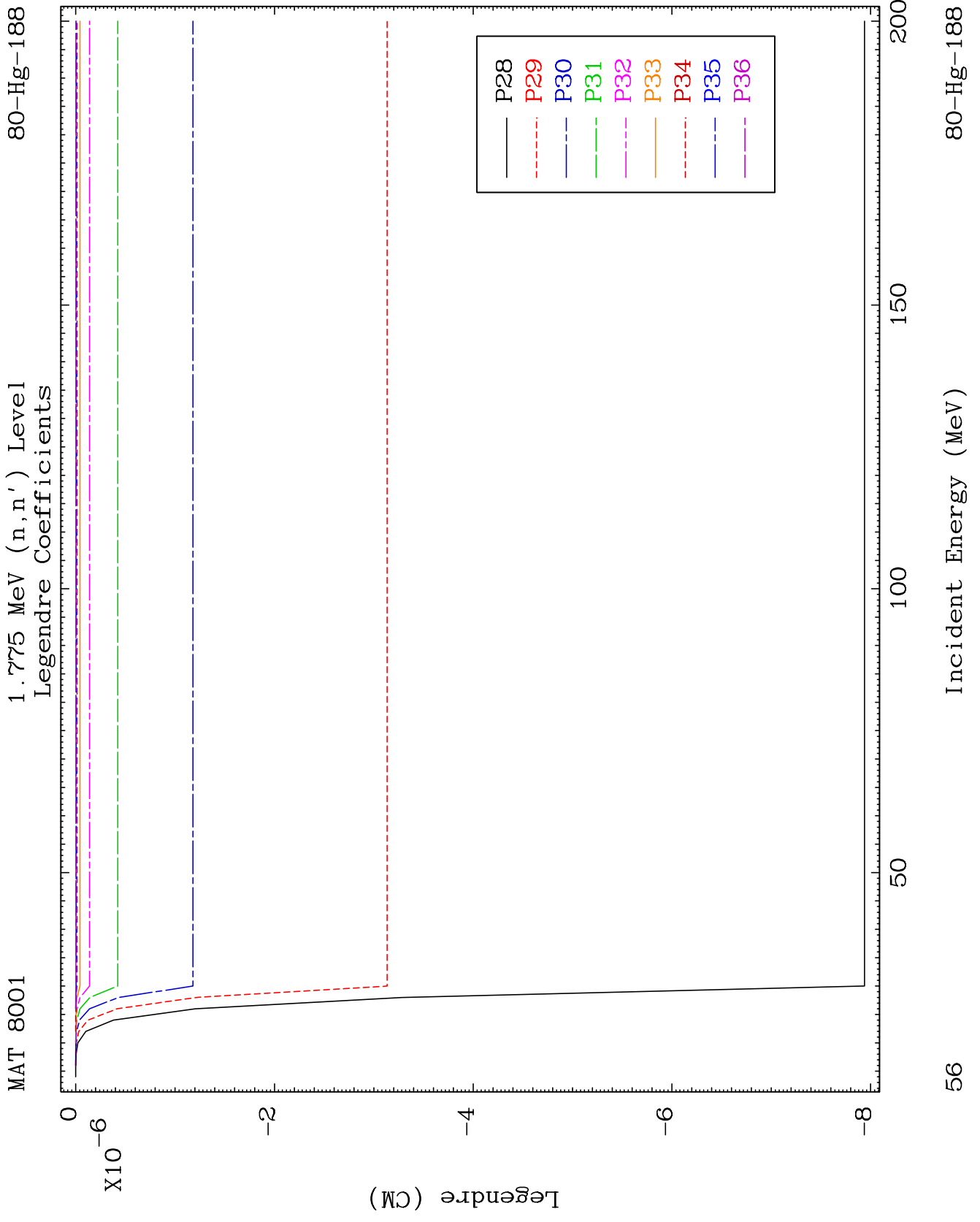


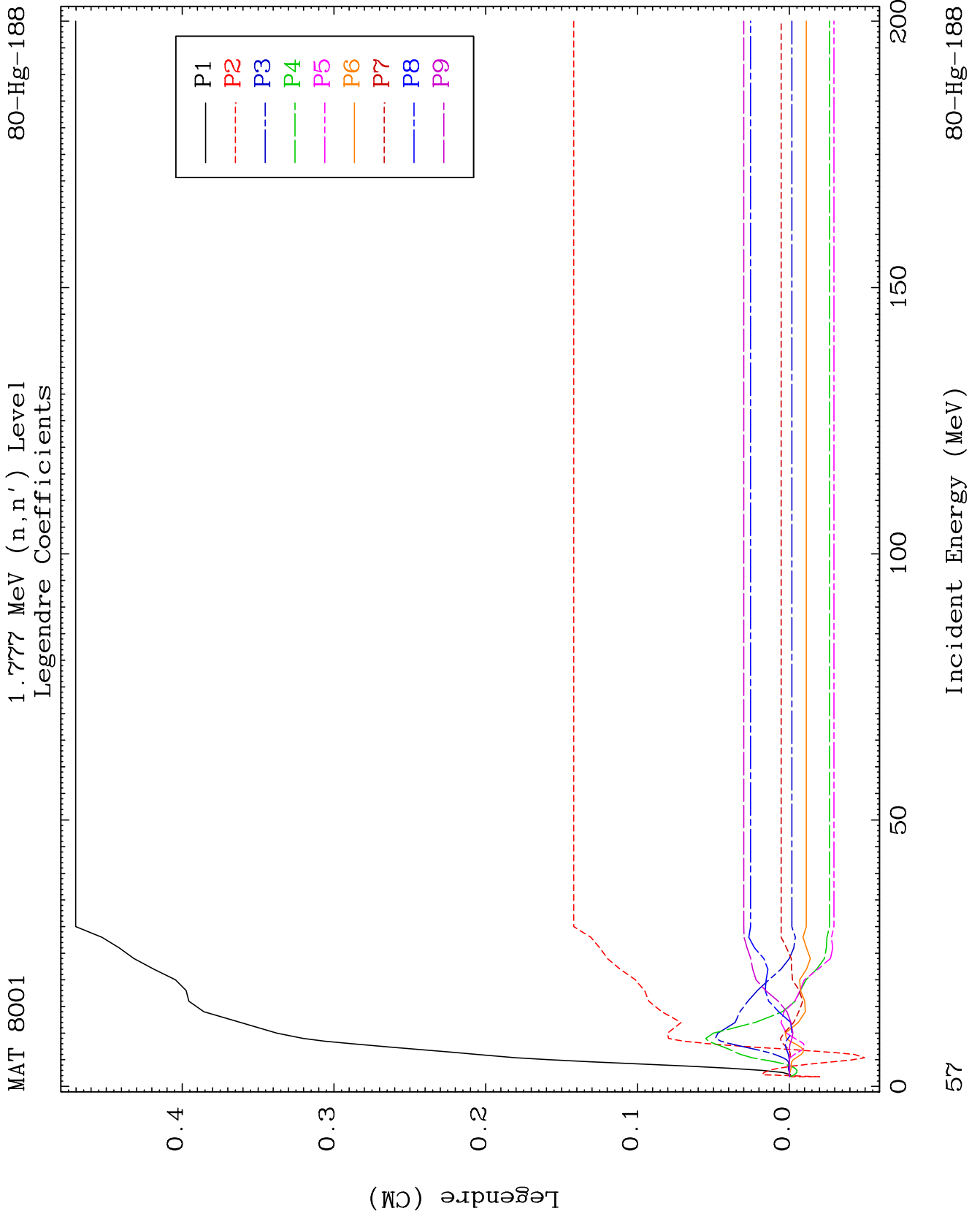




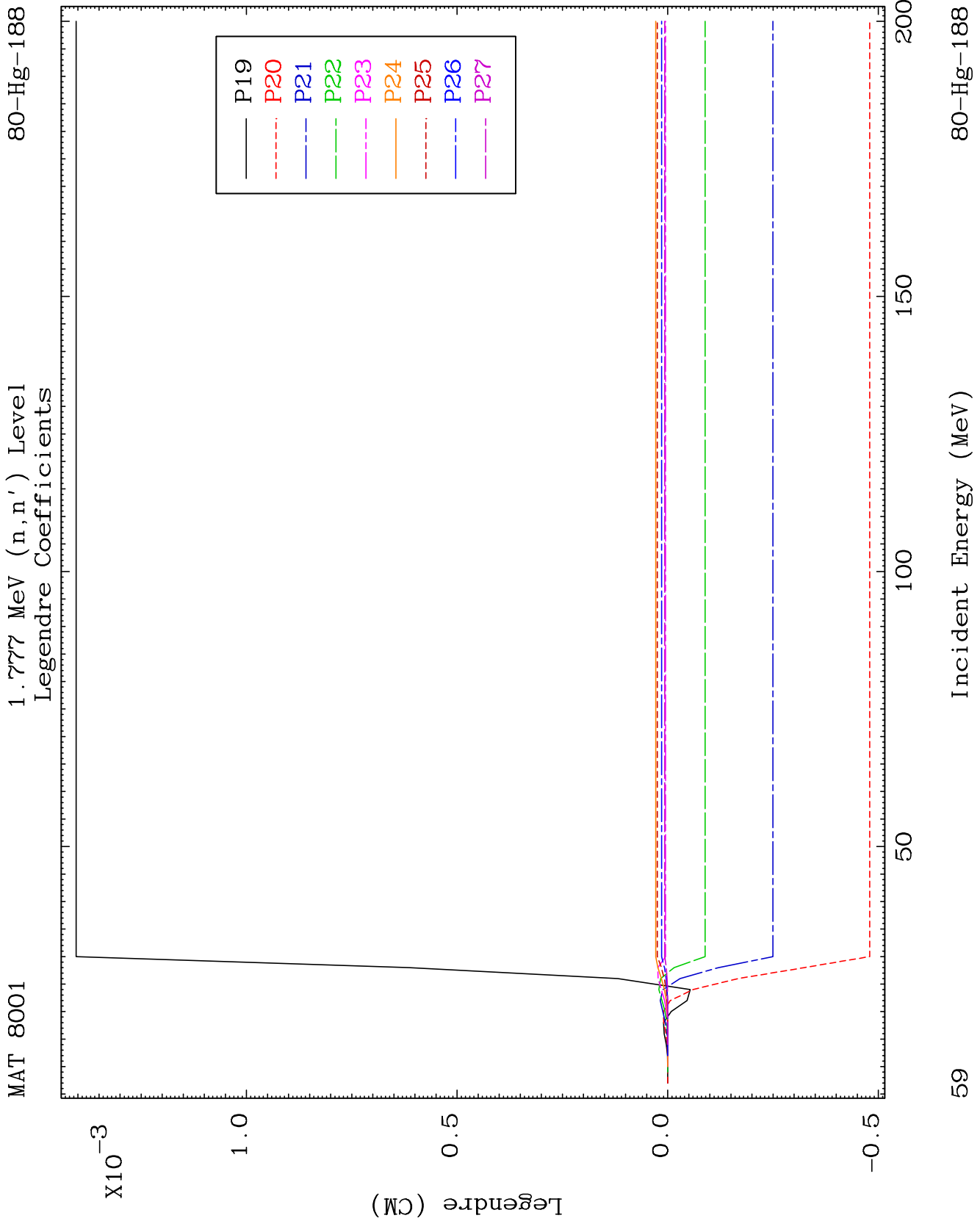


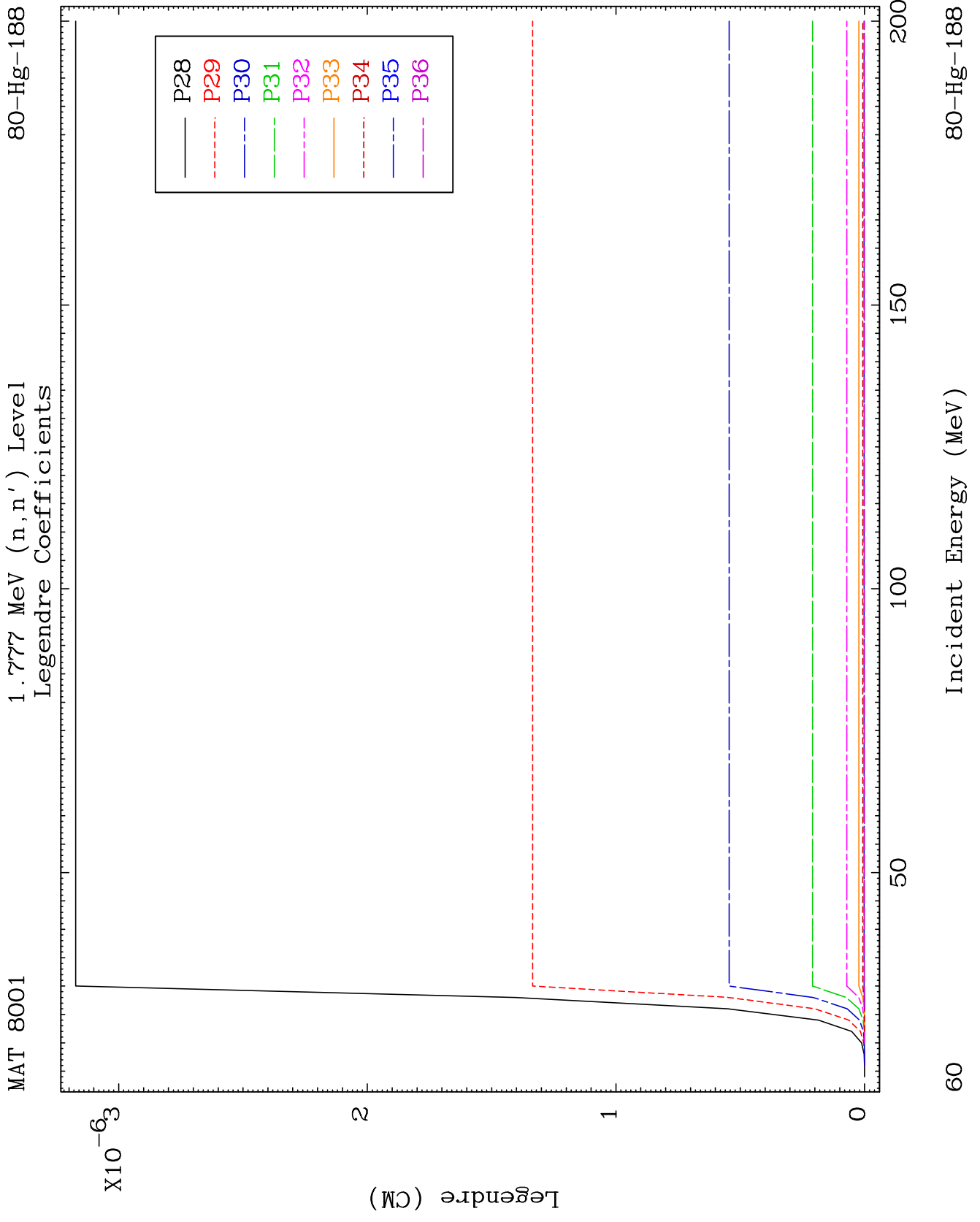


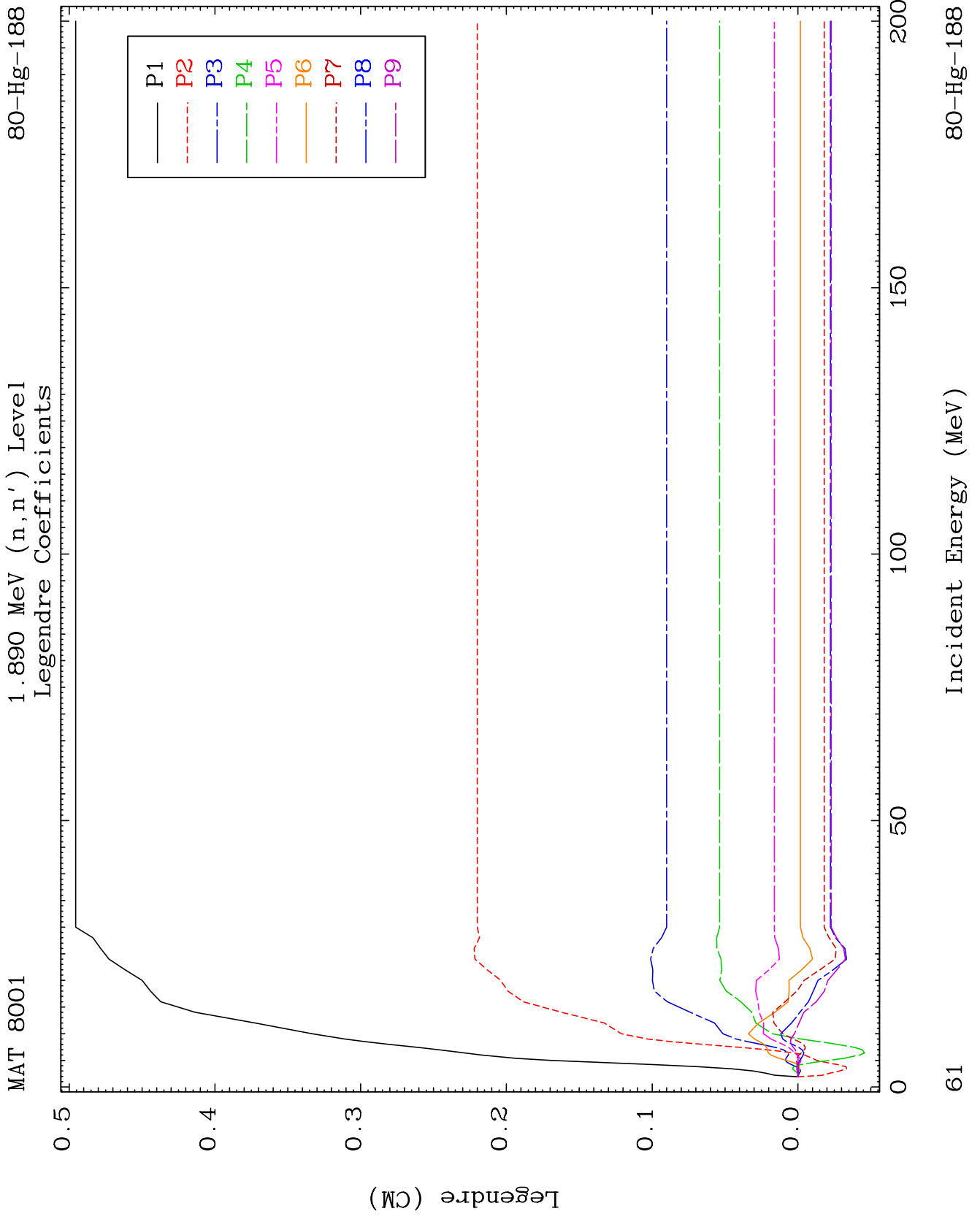


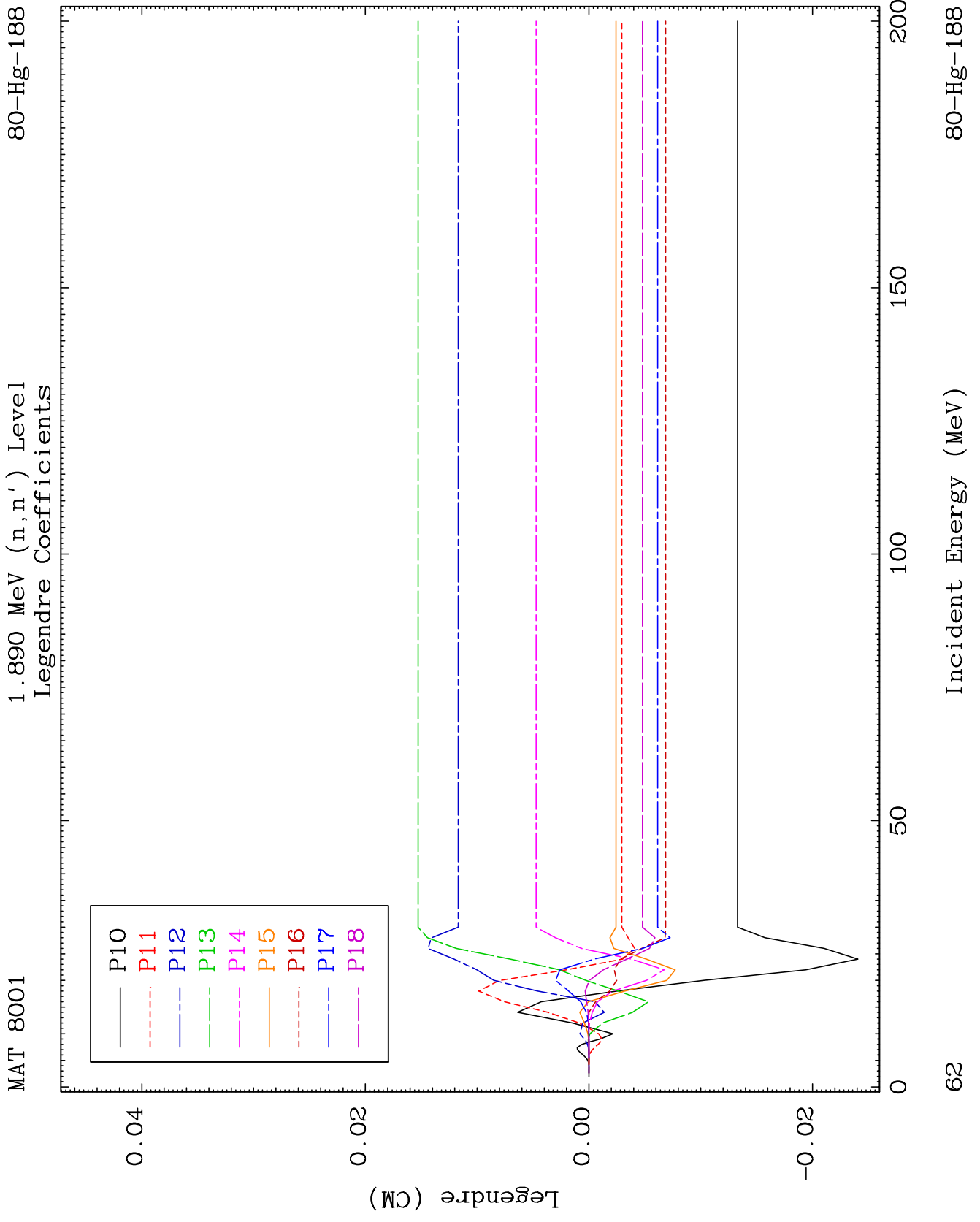


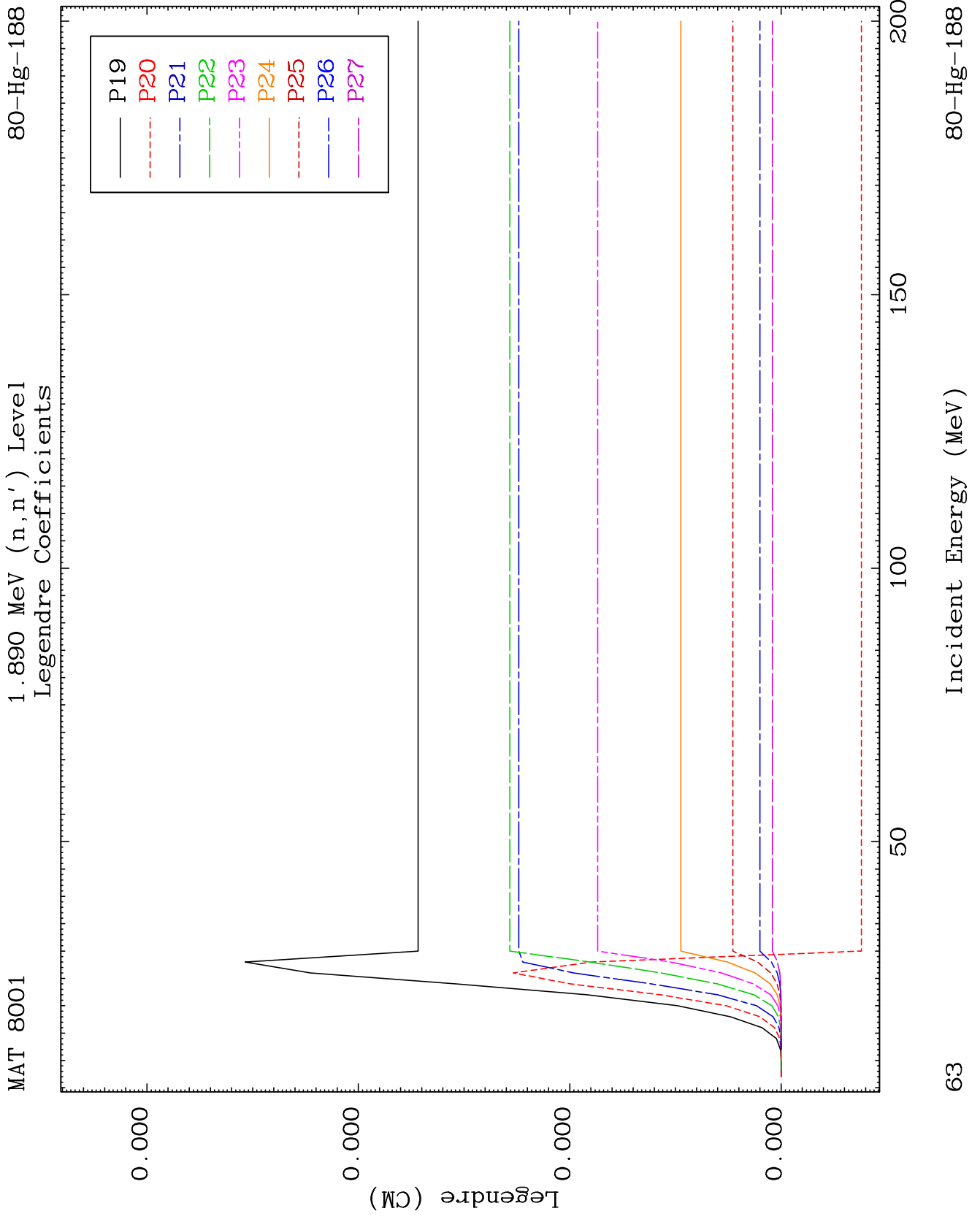




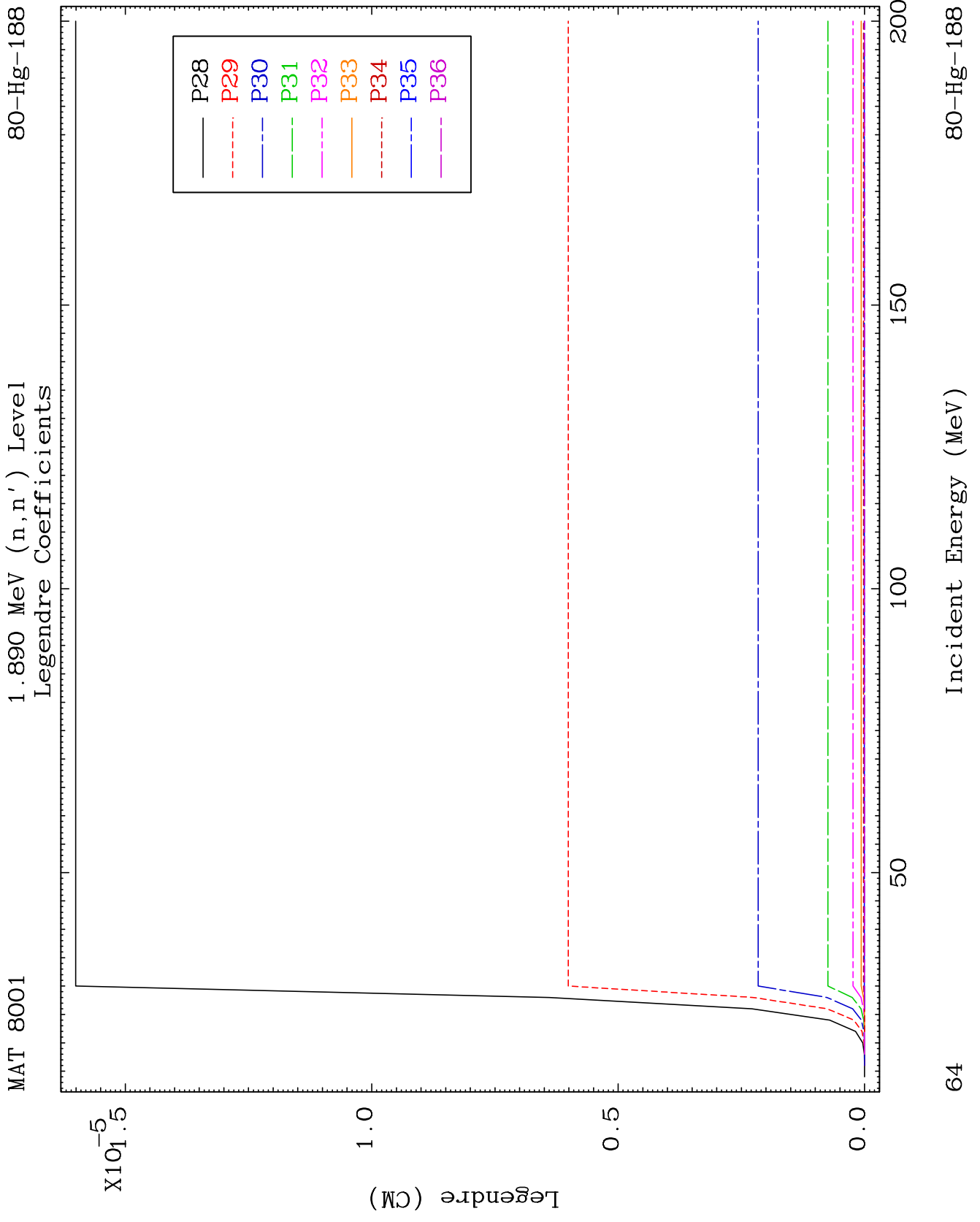


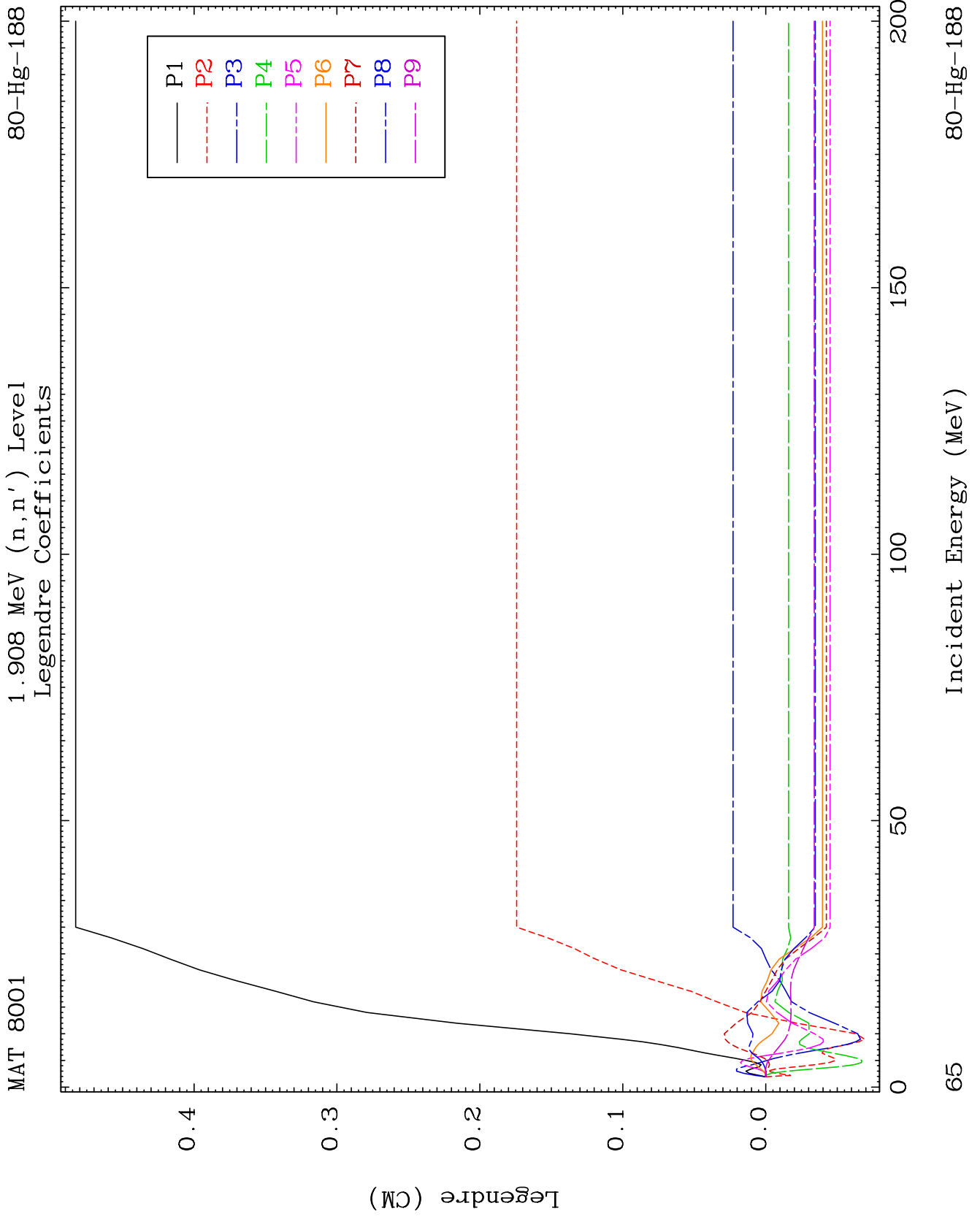










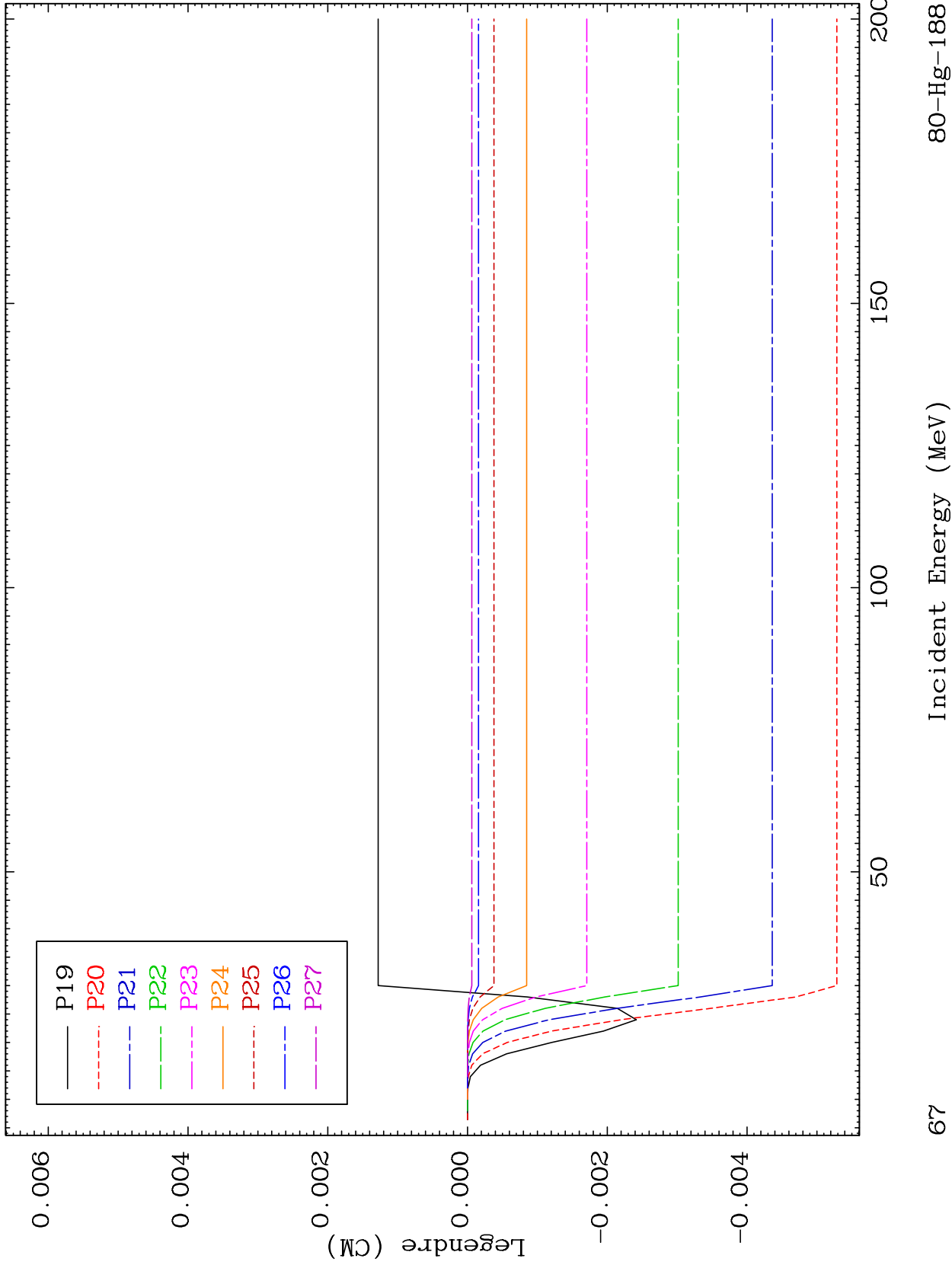




MAT 8001

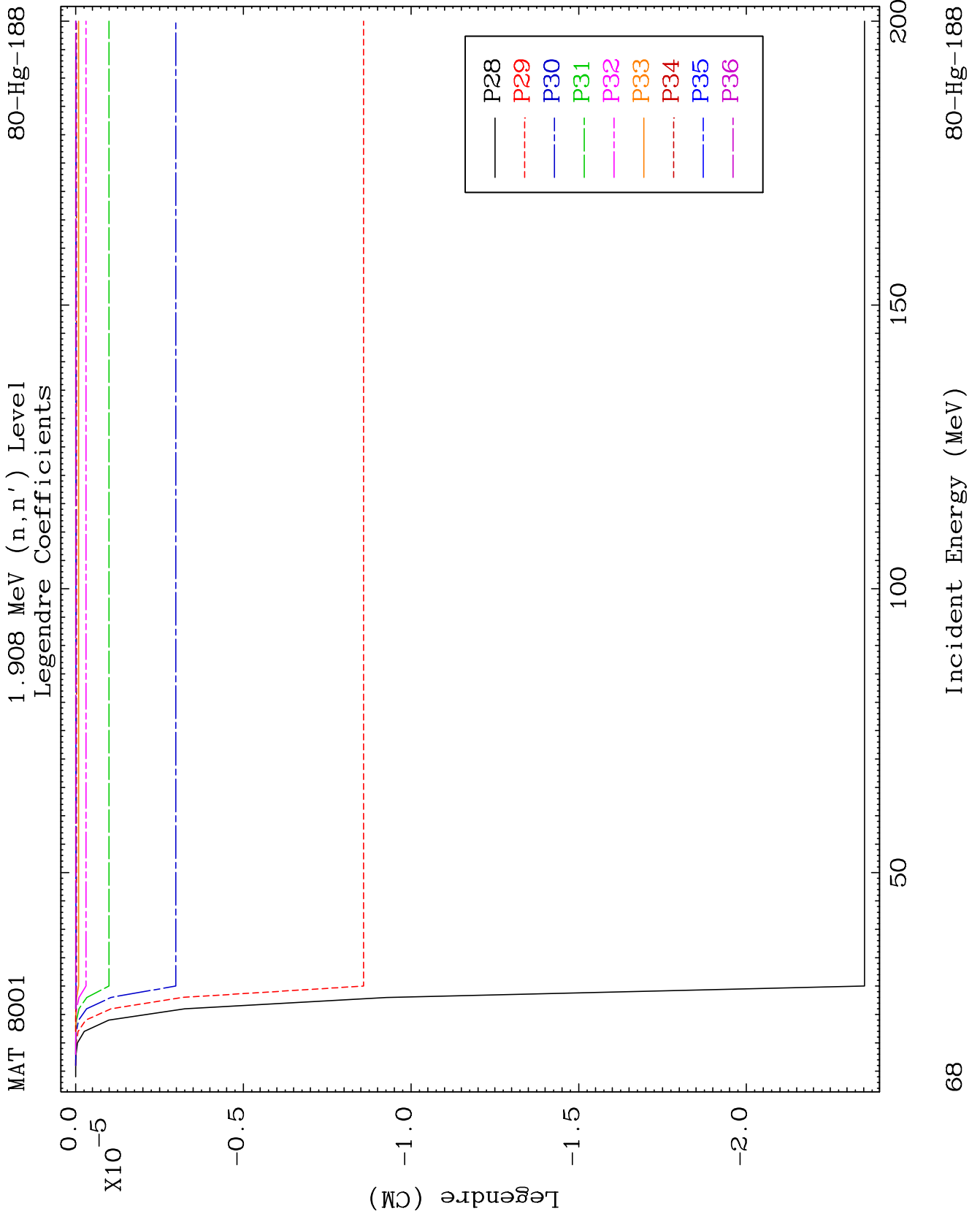
1.908 MeV (n,n') Level  
Legendre Coefficients

80-Hg-188

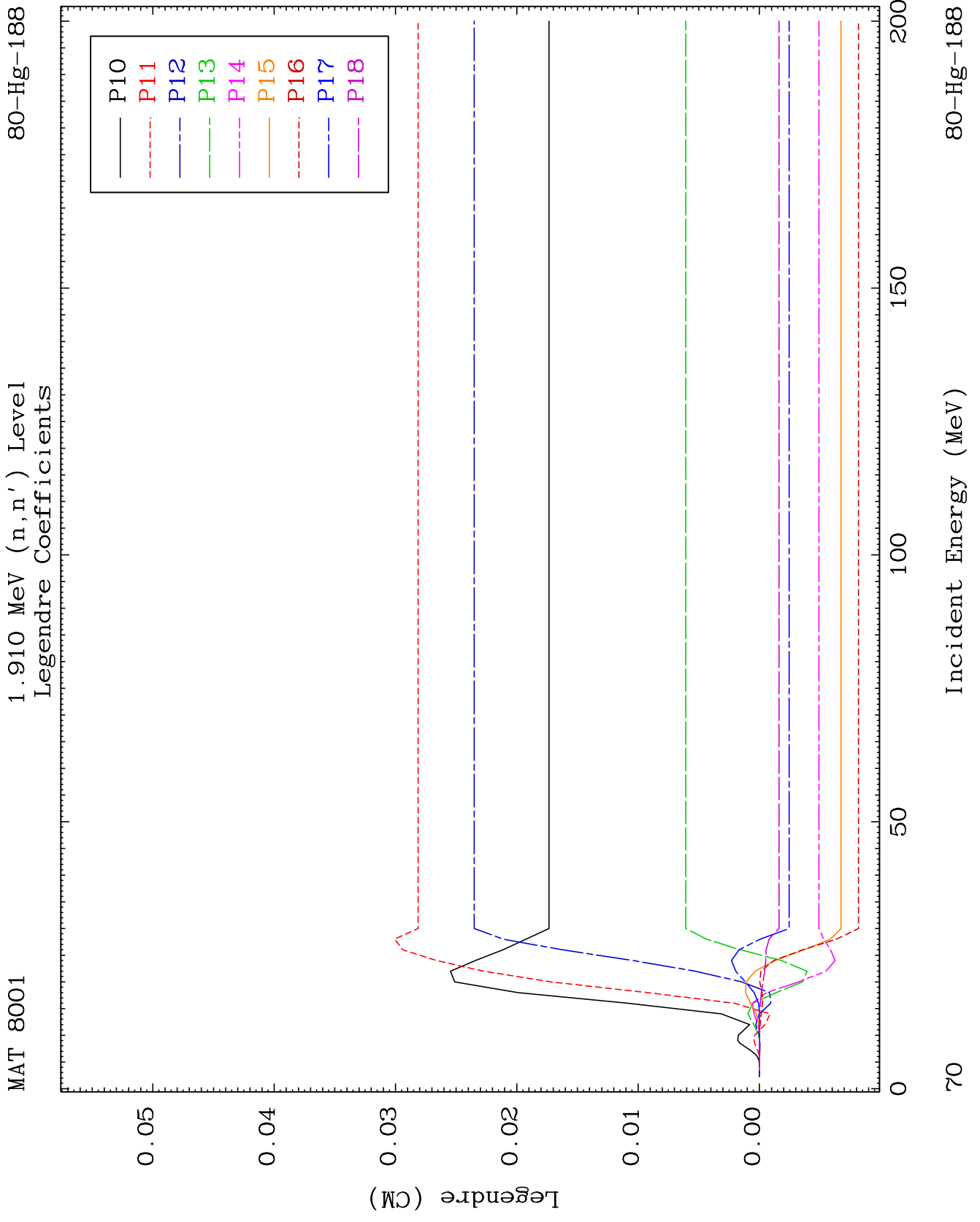


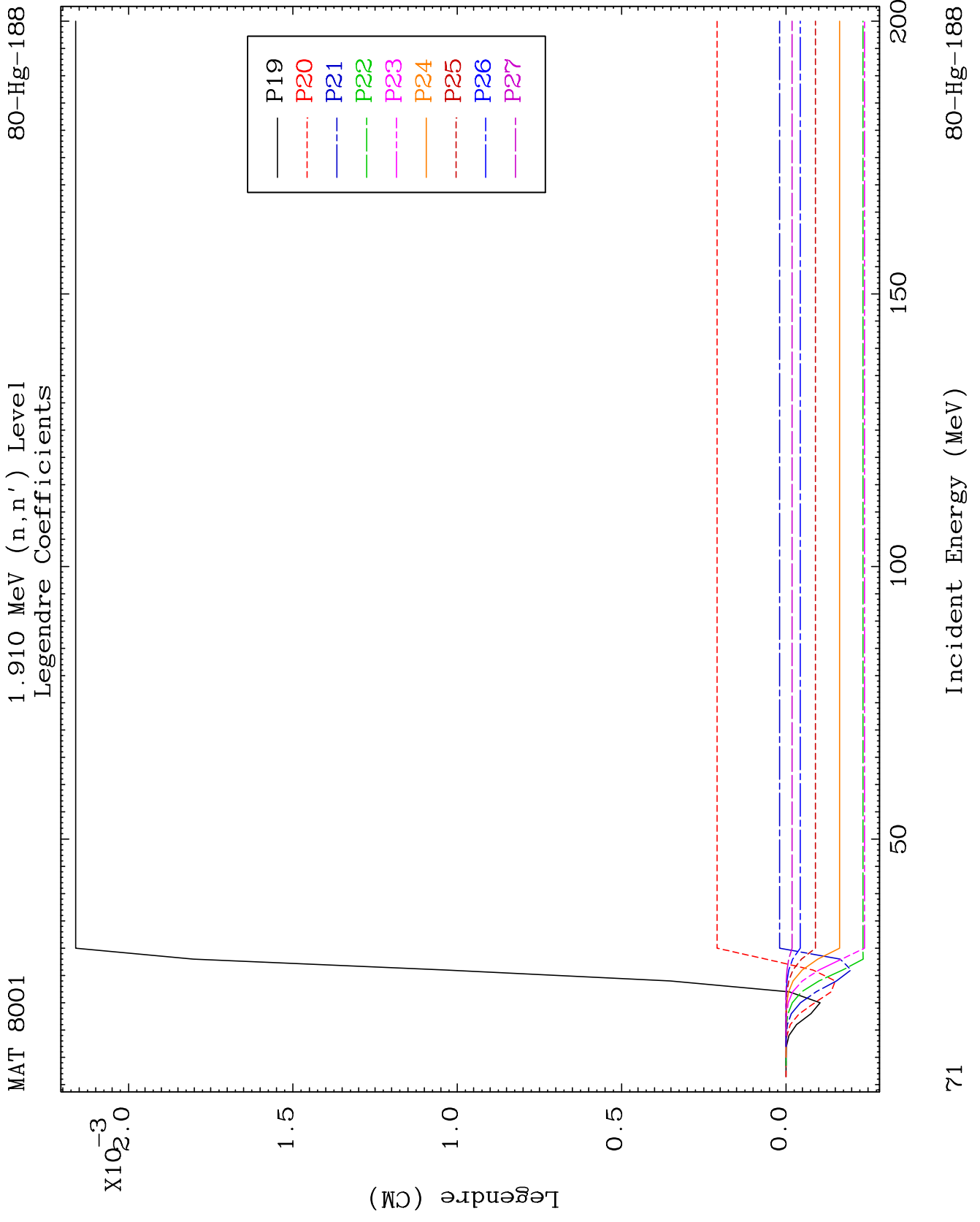
67

80-Hg-188

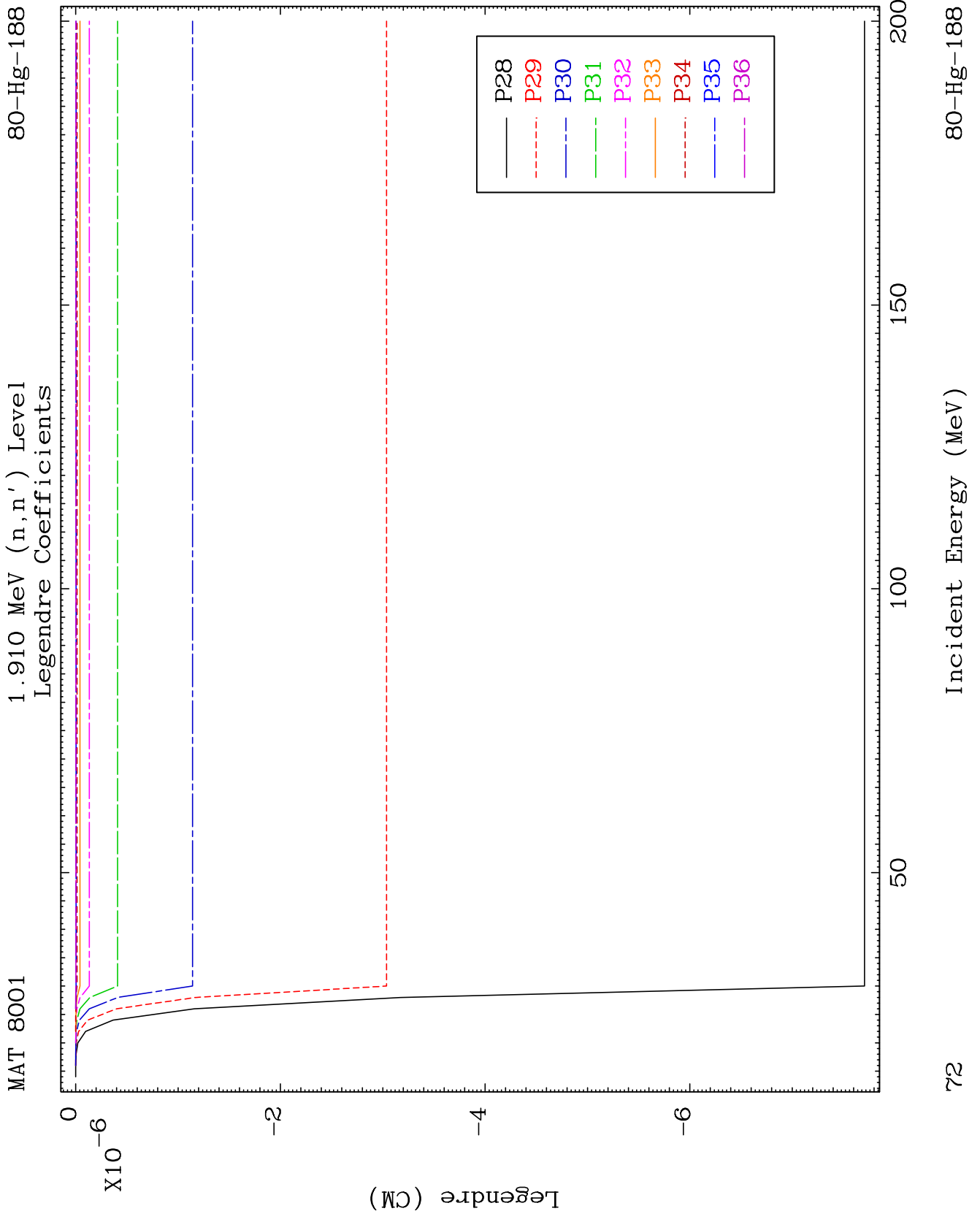


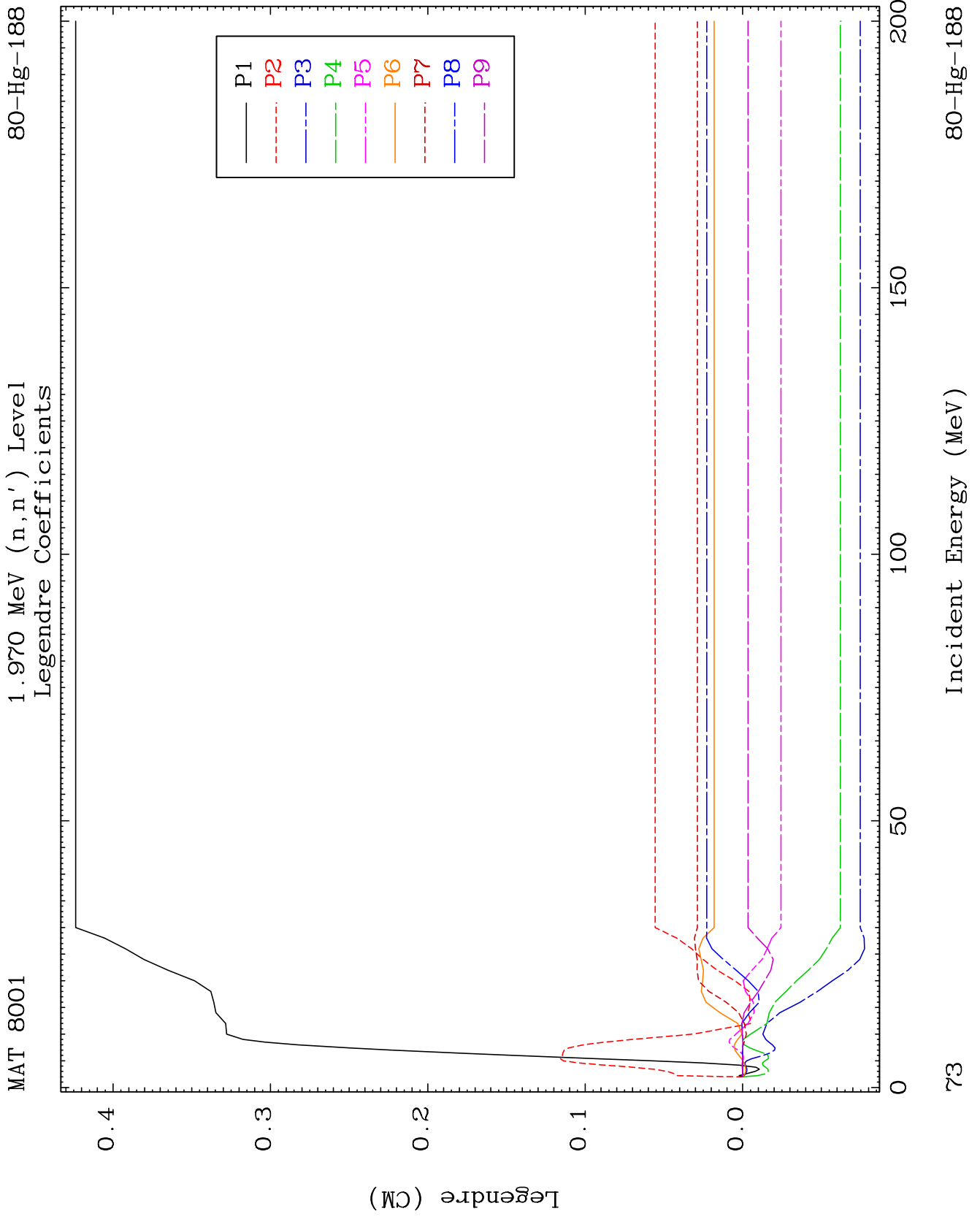




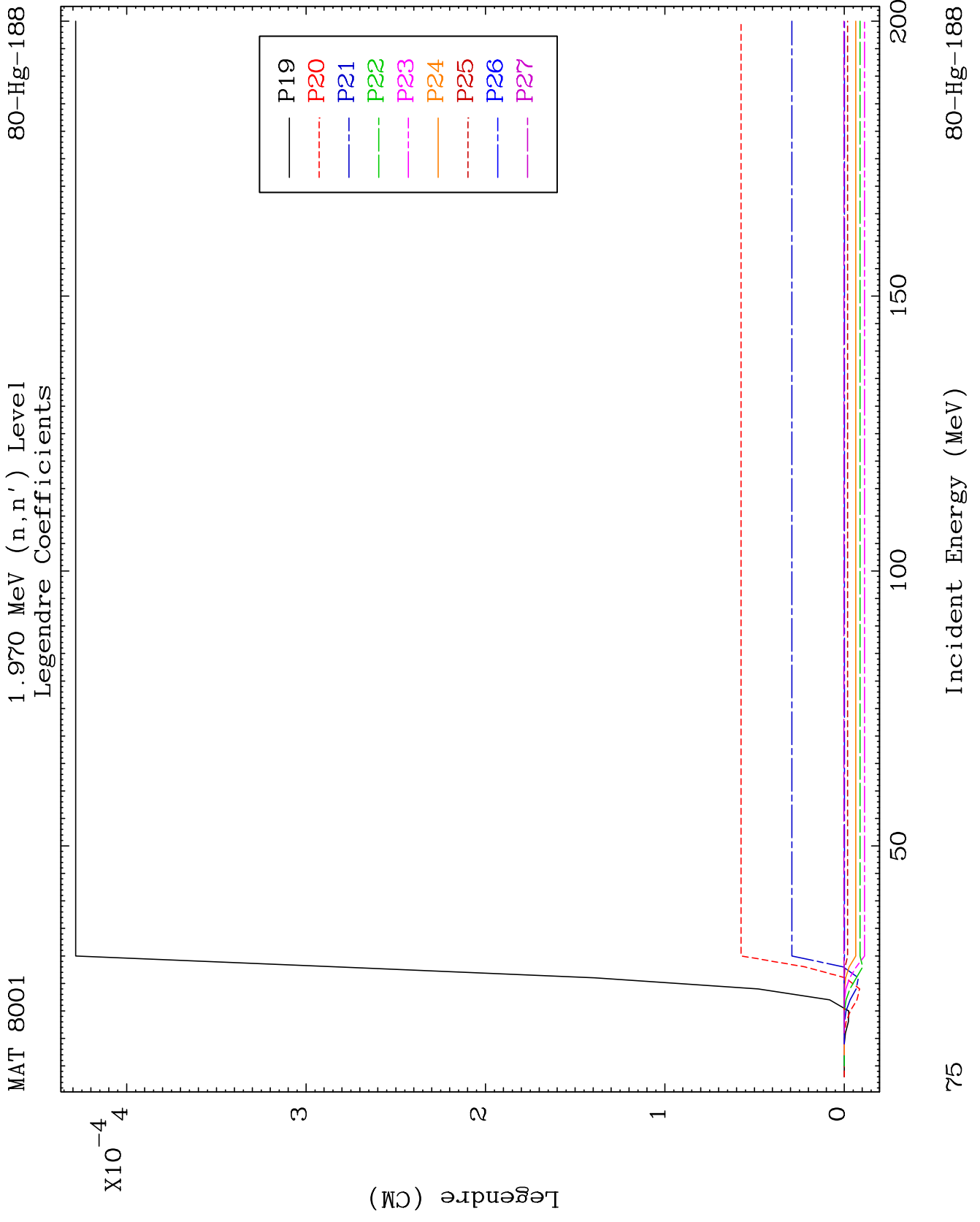


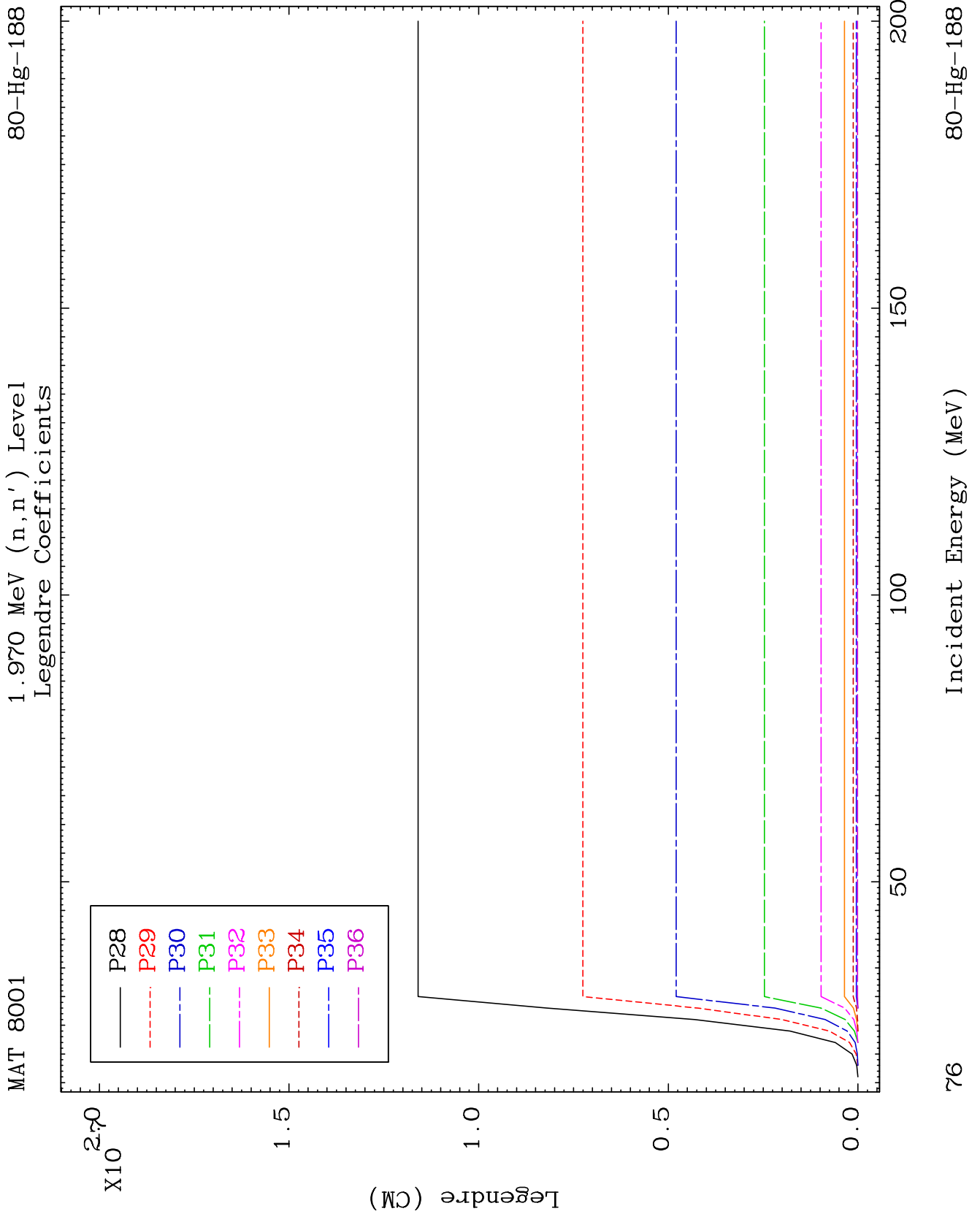


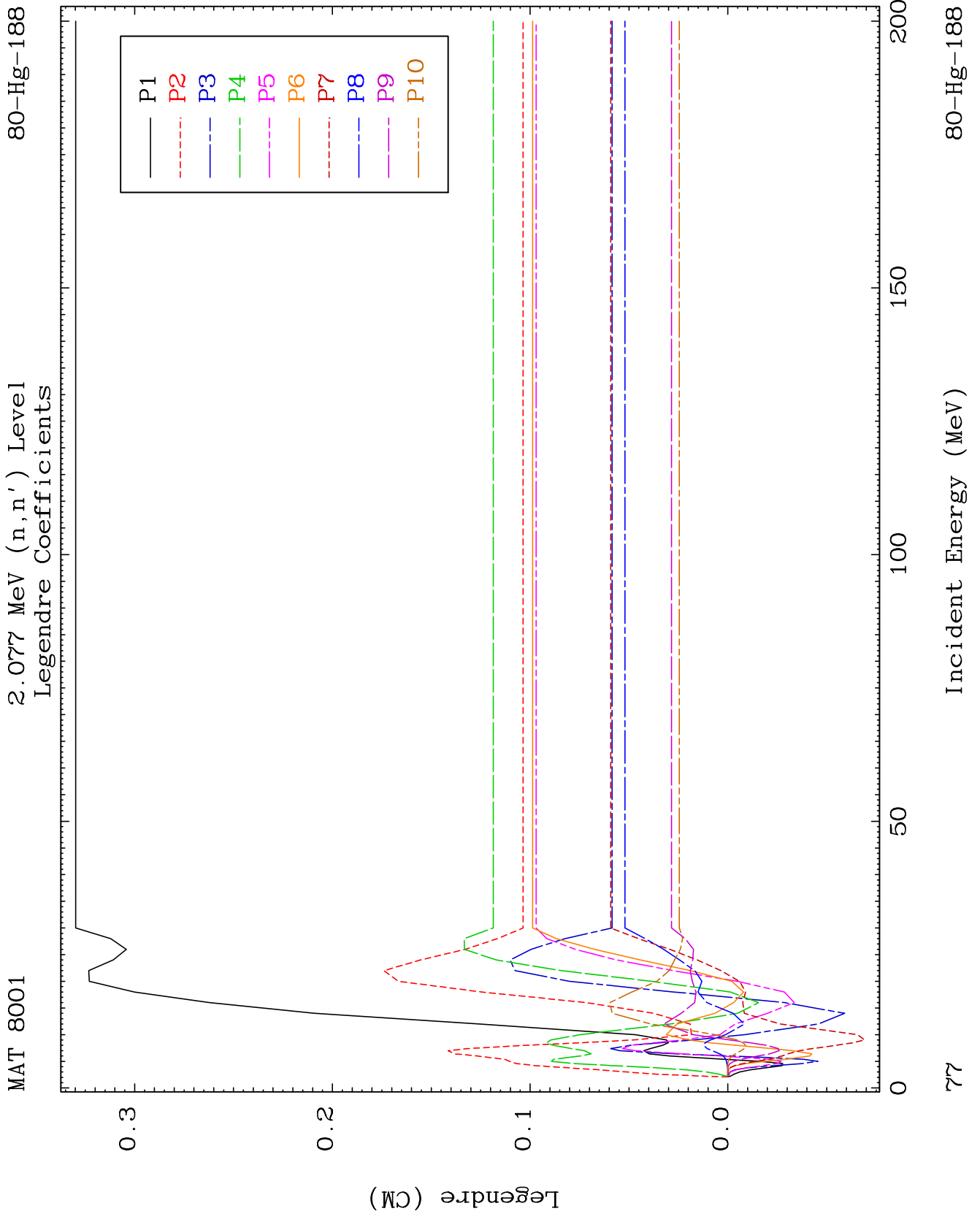


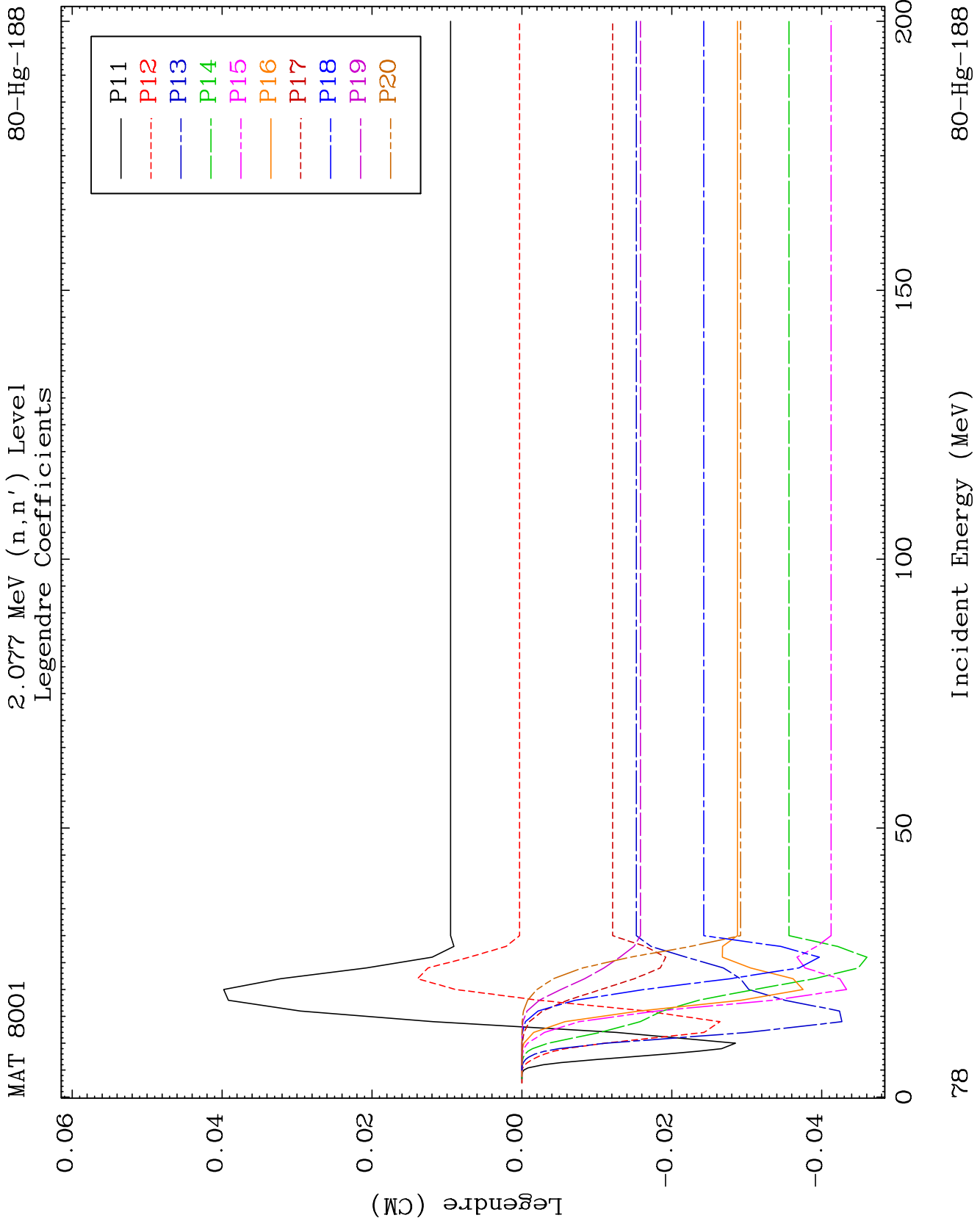


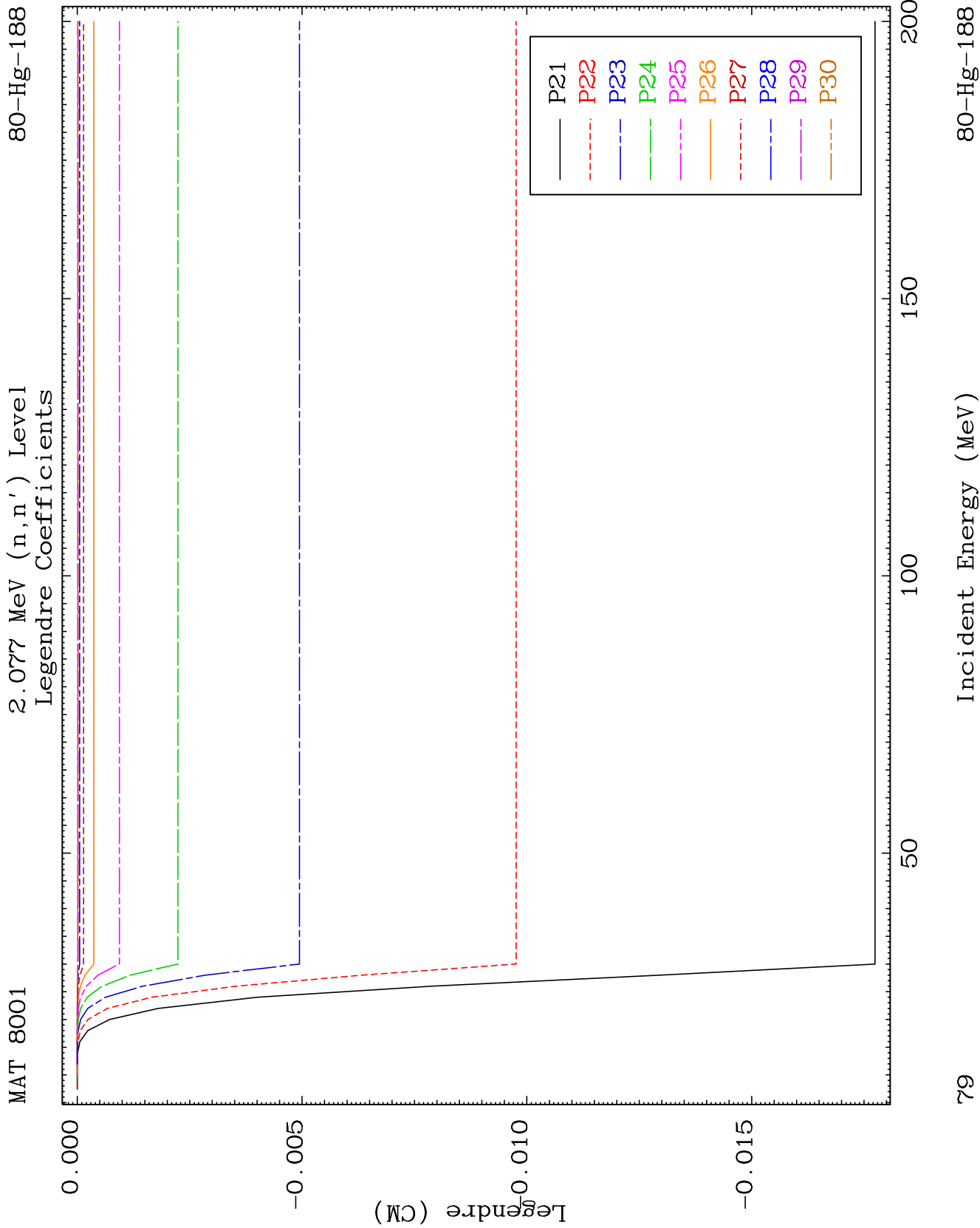




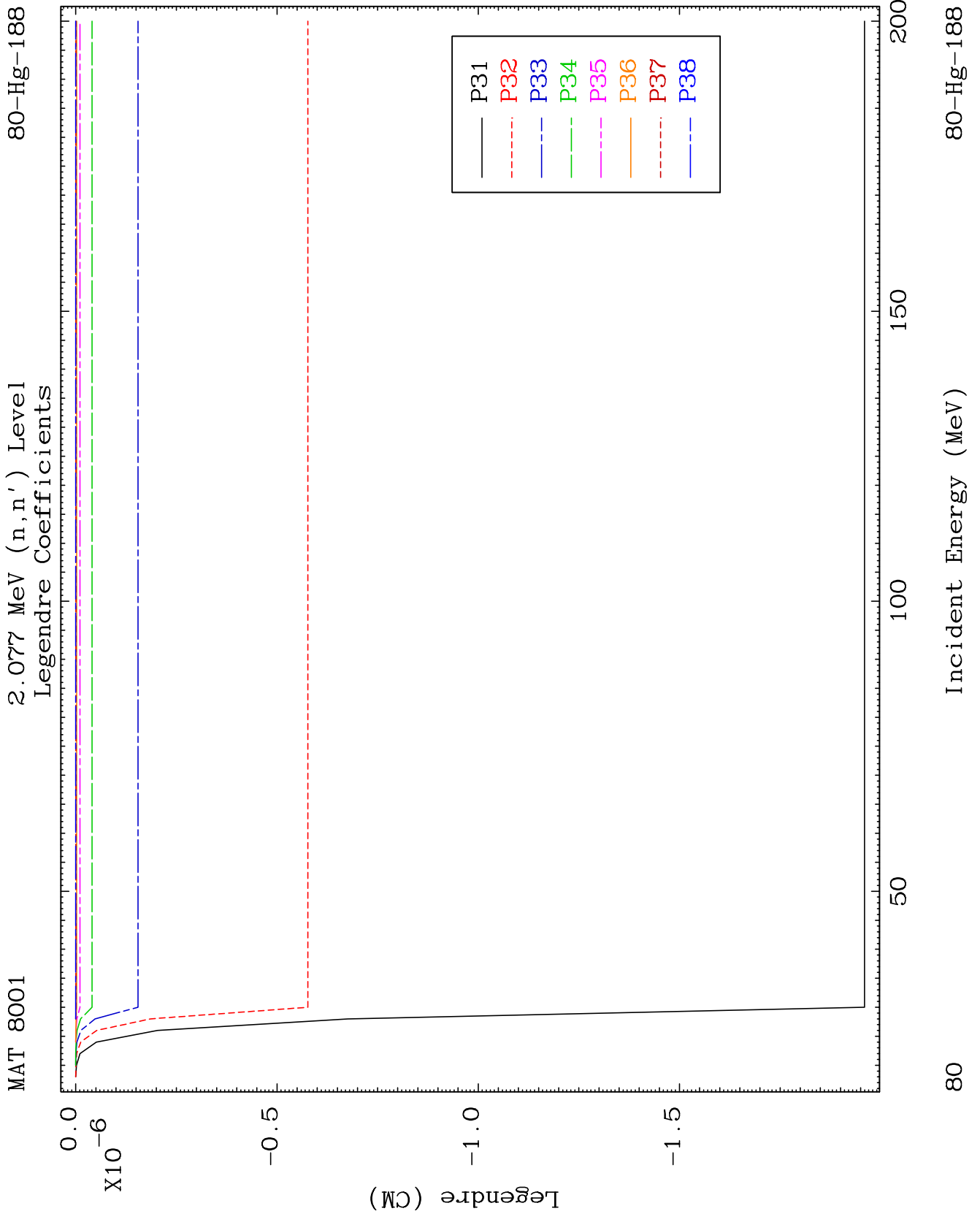










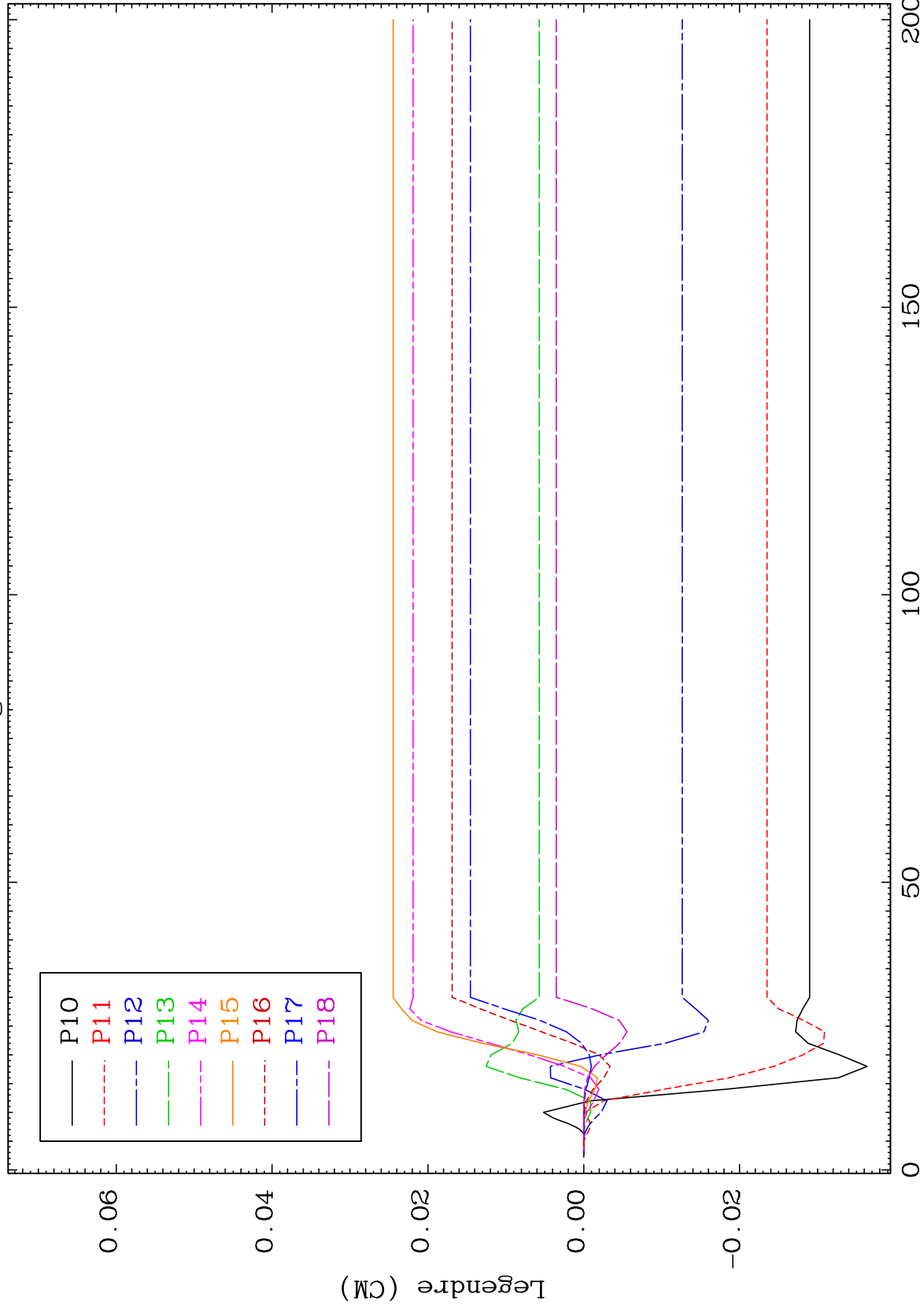




MAT 8001

2.136 MeV (n,n') Level  
Legendre Coefficients

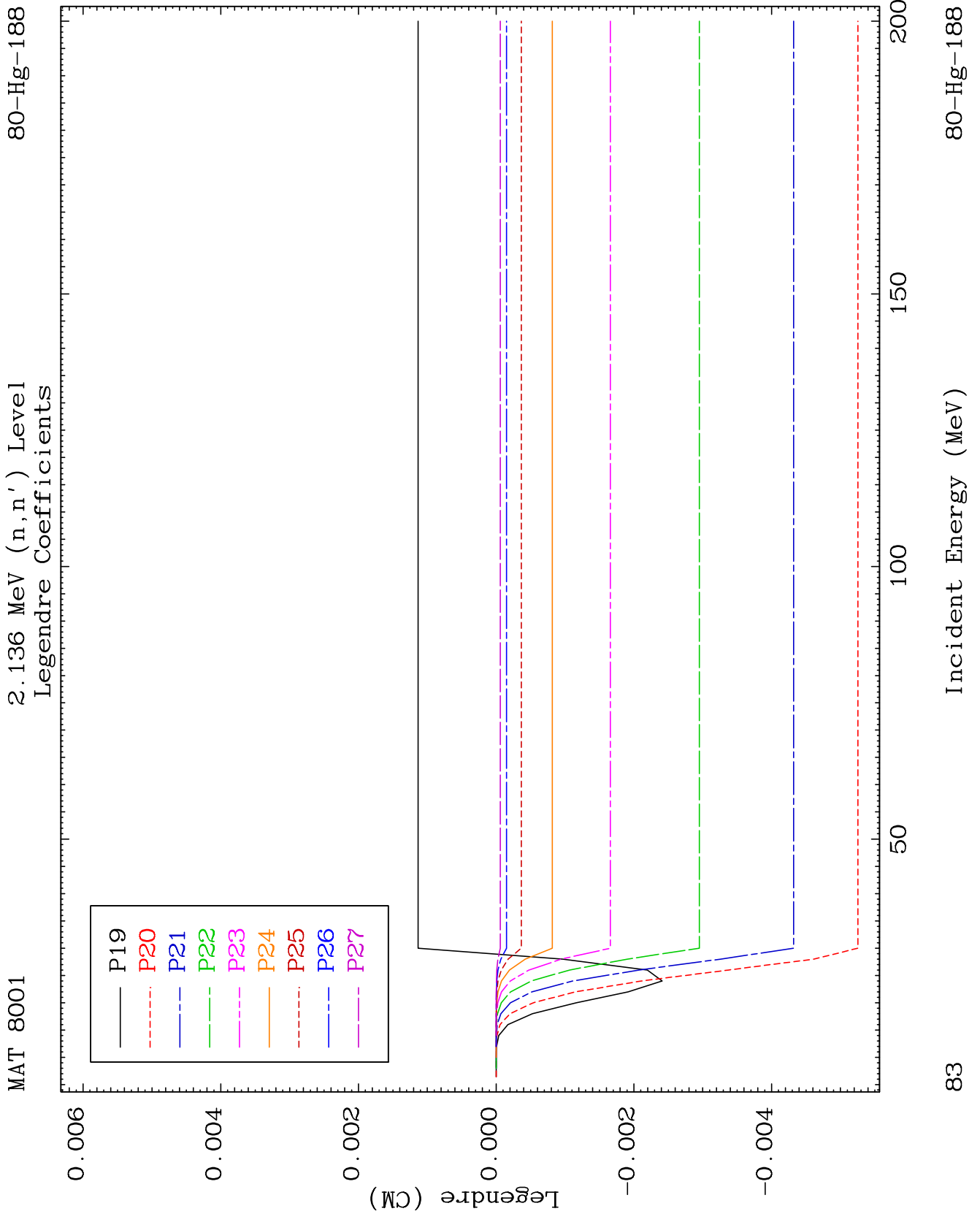
80-Hg-188

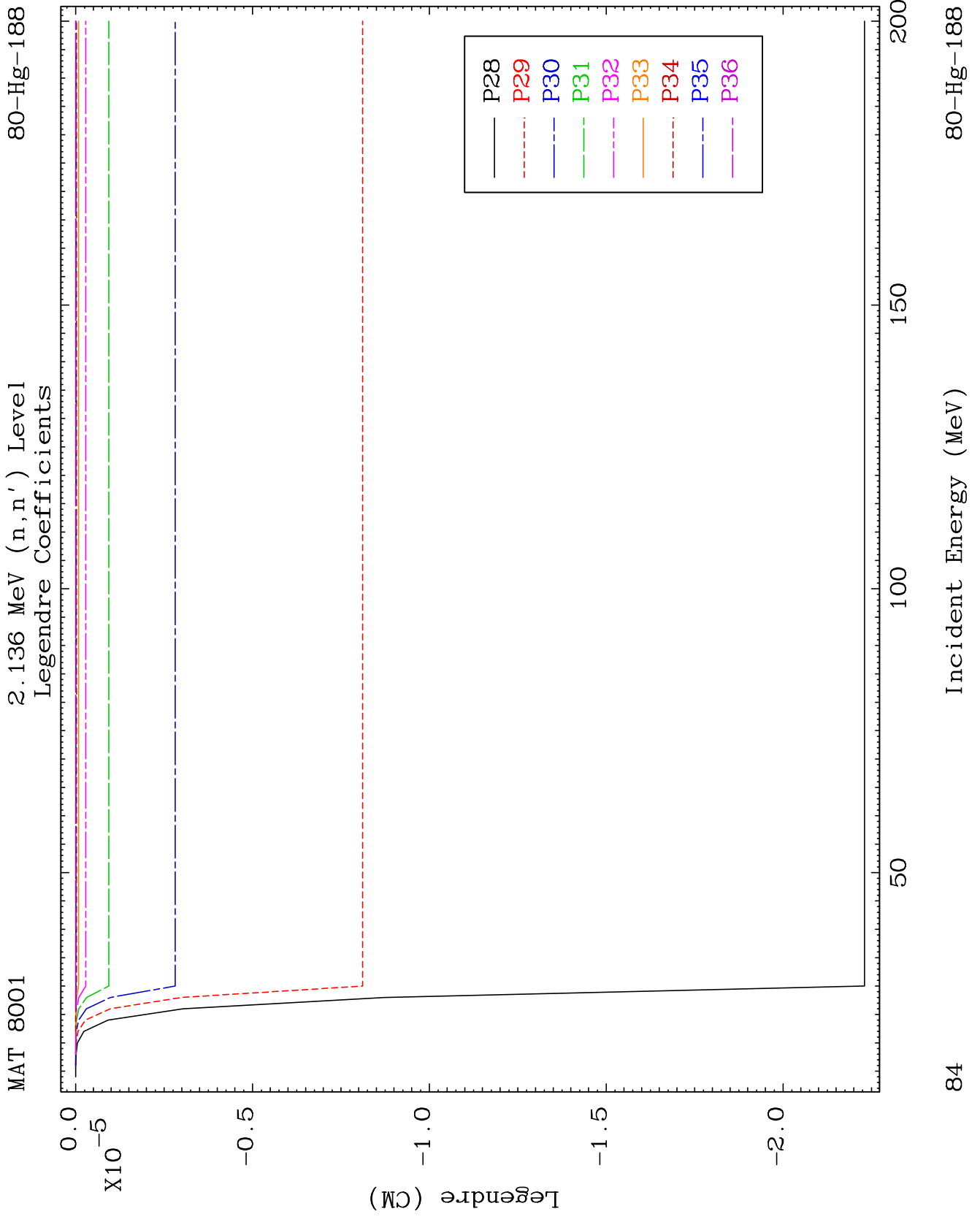


82

Incident Energy (MeV)

80-Hg-188



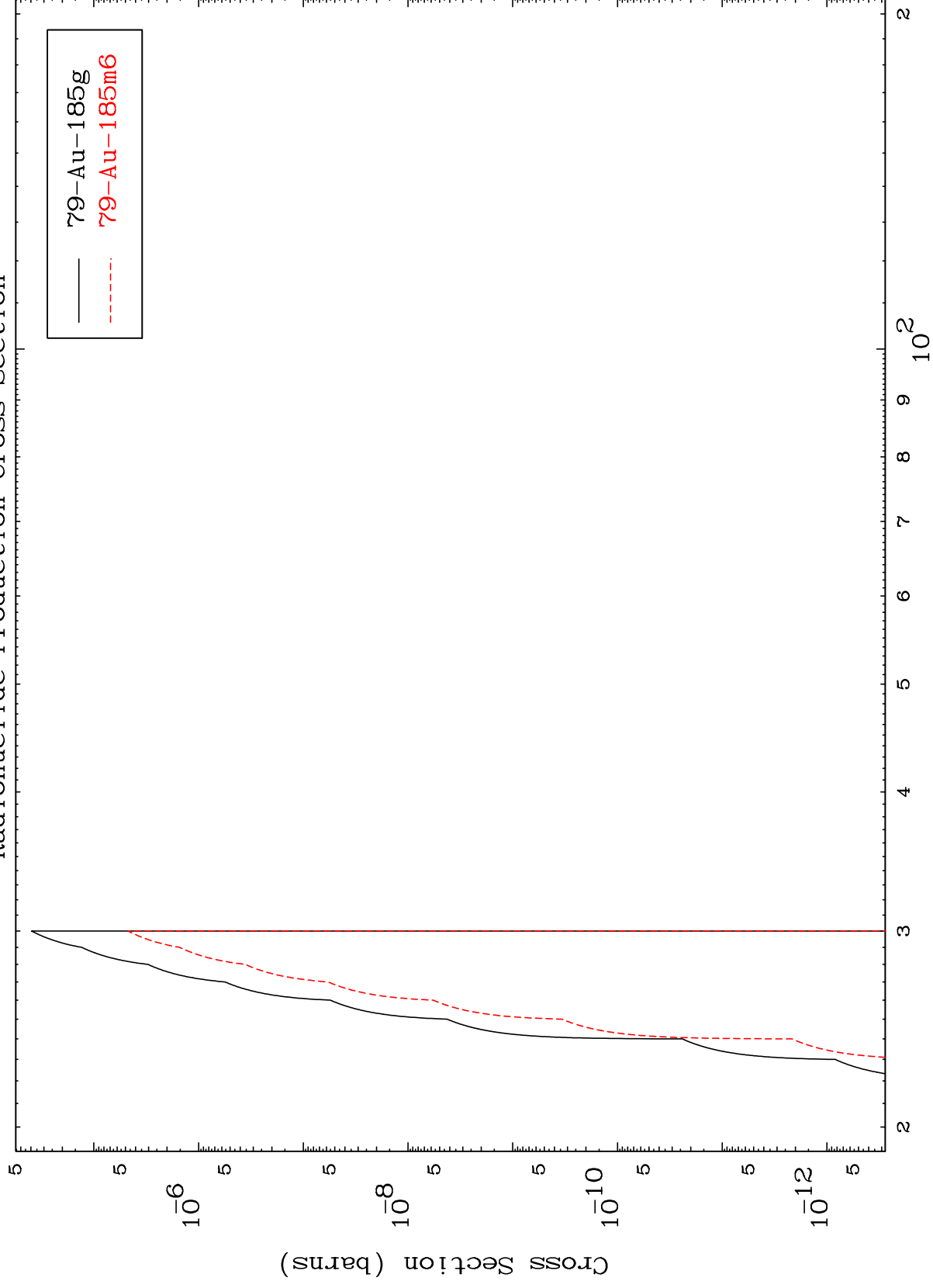


MAT 8001

(n,2n) d

80-Hg-188

Radionuclide Production Cross Section



85

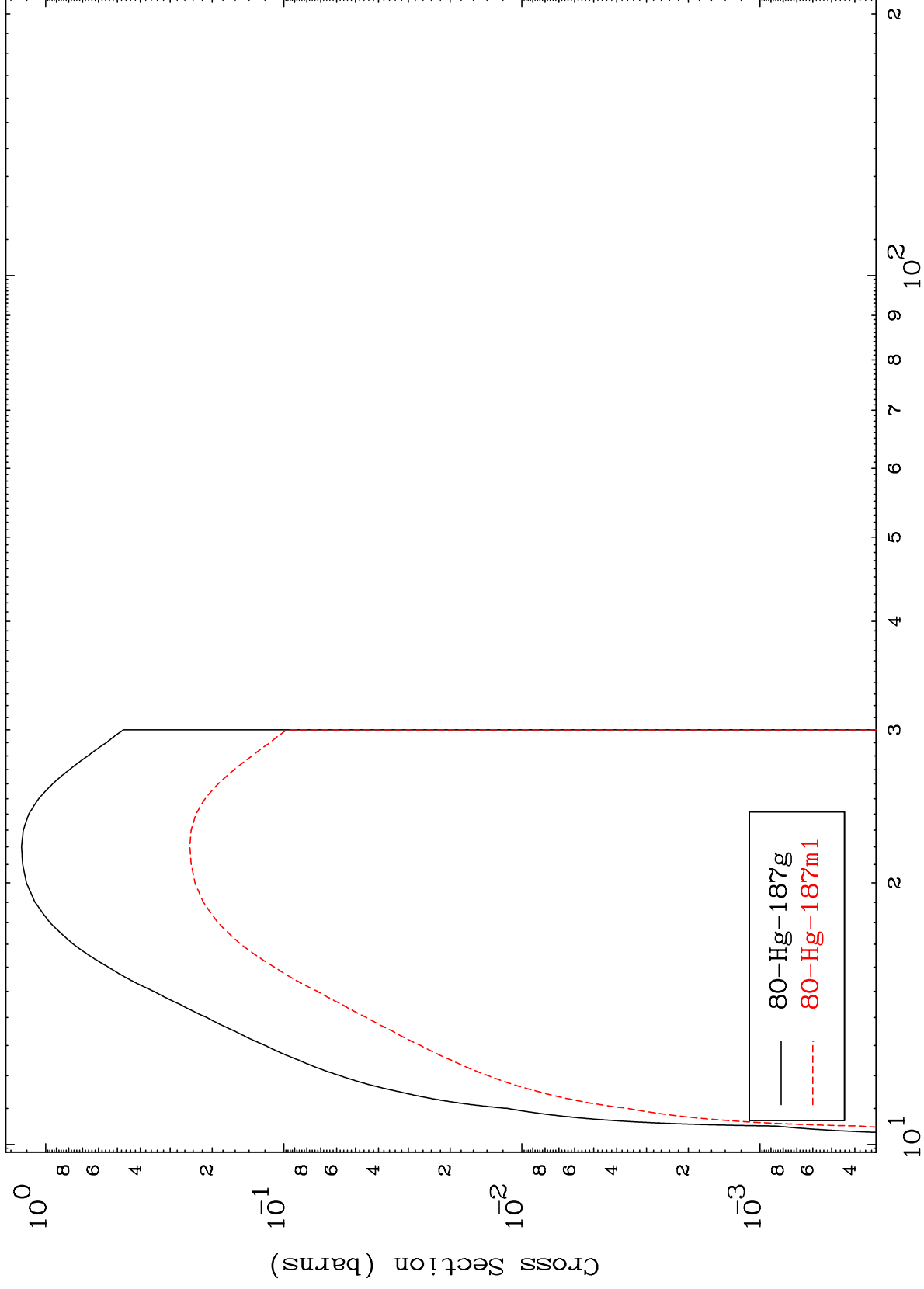
Incident Energy (MeV)

80-Hg-188

MAT 8001

80-Hg-188

(n,2n)  
Radionuclide Production Cross Section



80-Hg-188

Incident Energy (MeV)

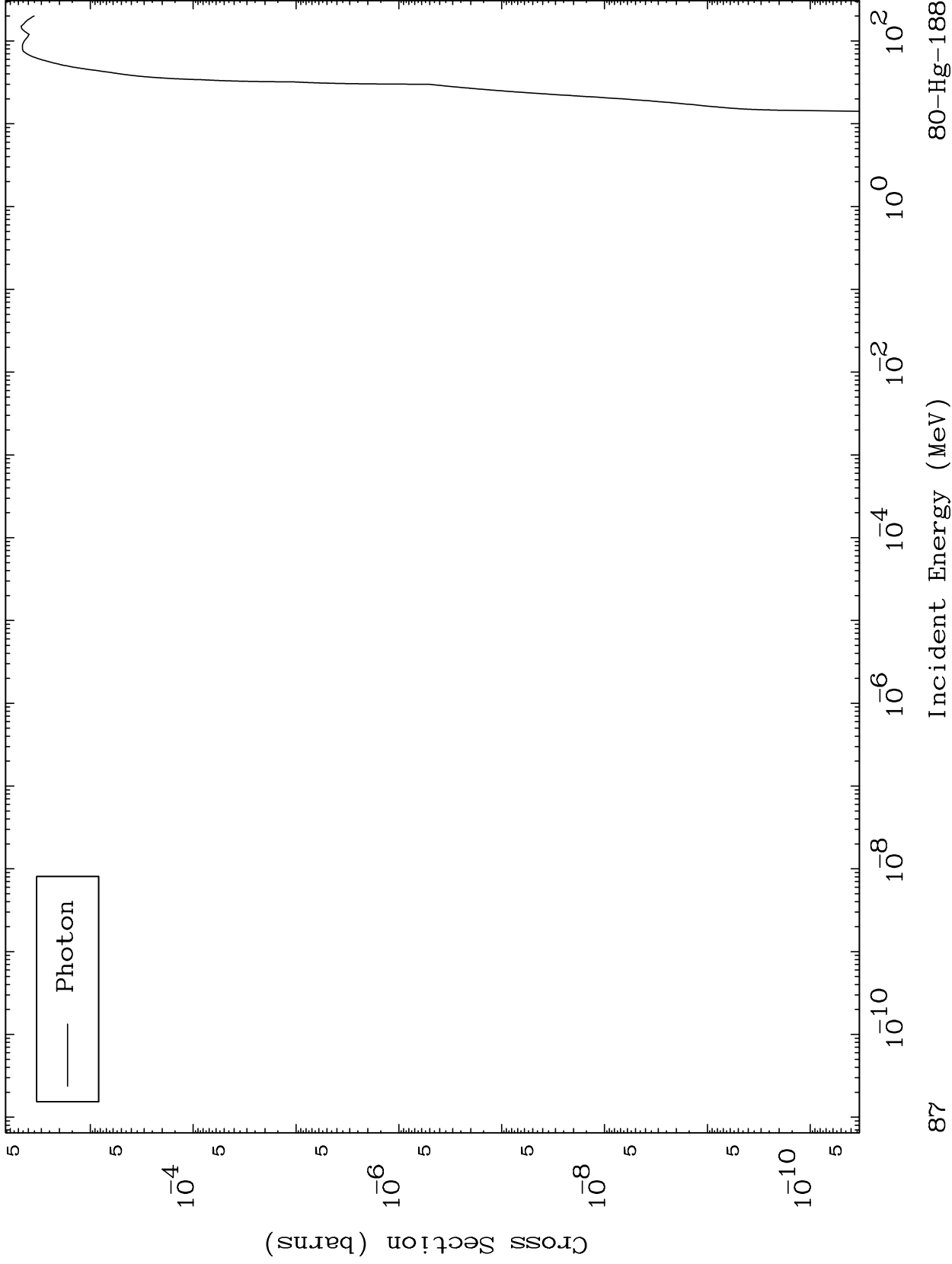
86

MAT 8001

Fission

80-Hg-188

Radionuclide Production Cross Section



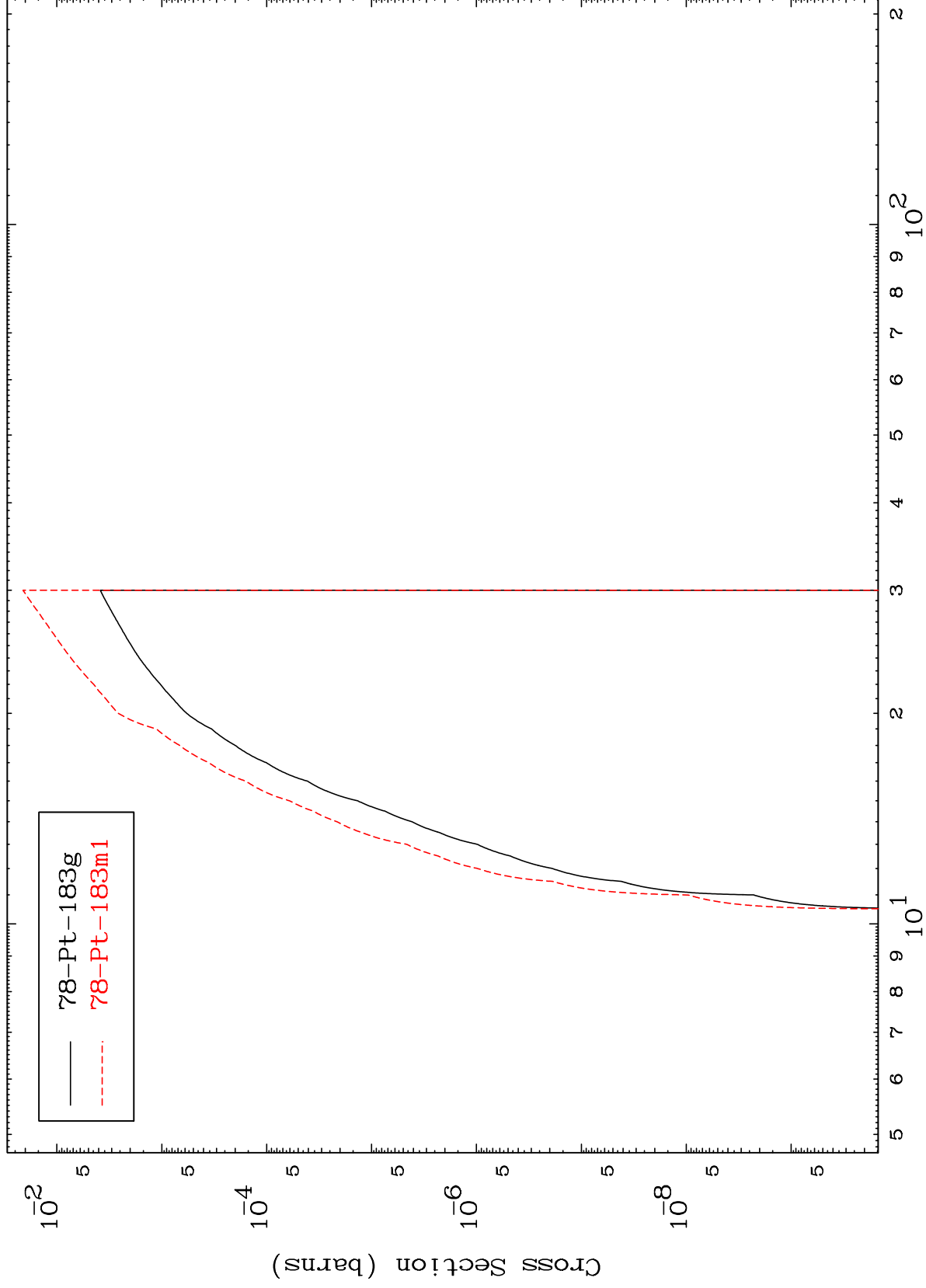


MAT 8001

$(n, 2n)$   $\alpha$

80-Hg-188

Radionuclide Production Cross Section



88

Incident Energy (MeV)

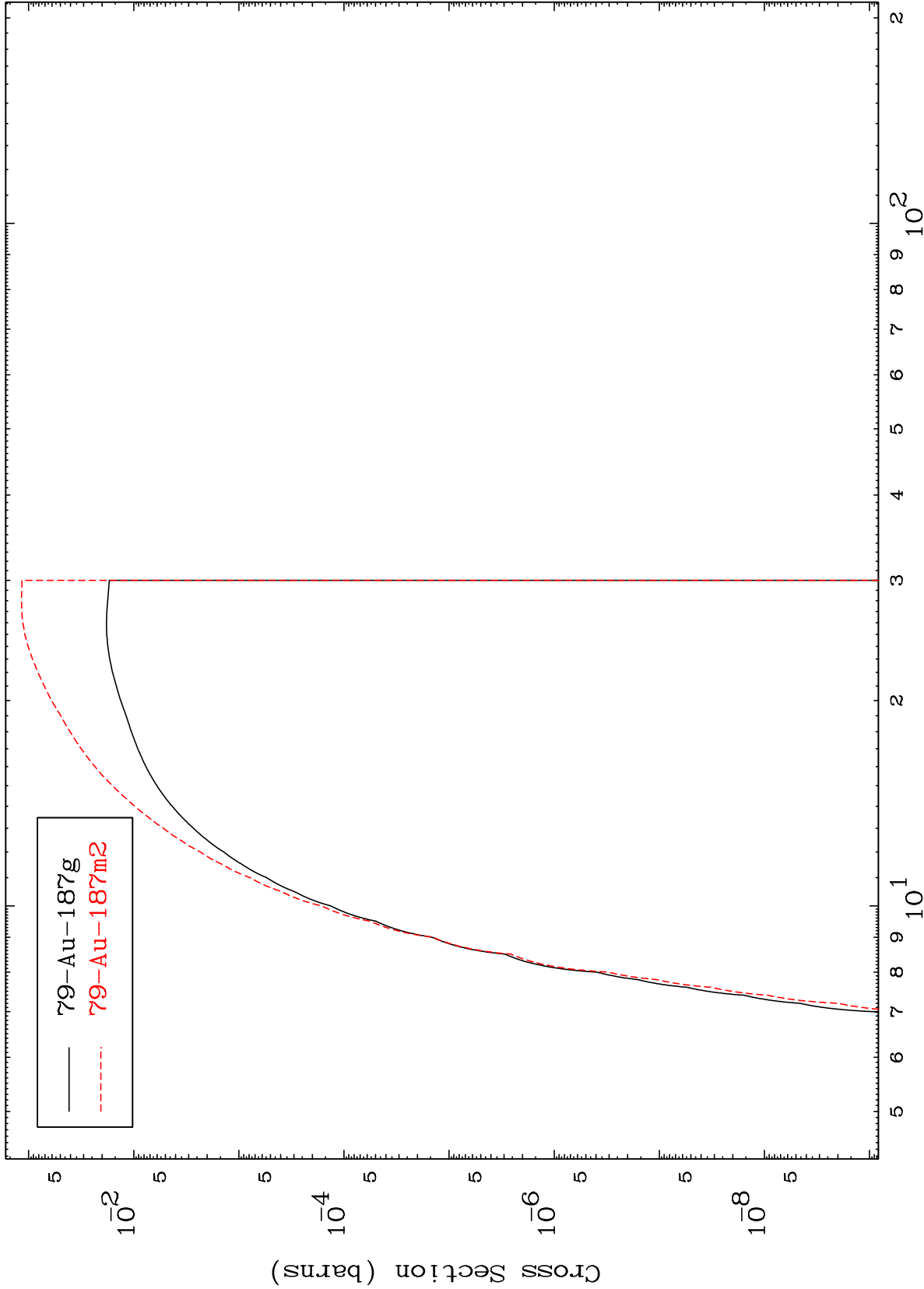
80-Hg-188

MAT 8001

(n,n') p

80-Hg-188

Radionuclide Production Cross Section



89

Incident Energy (MeV)

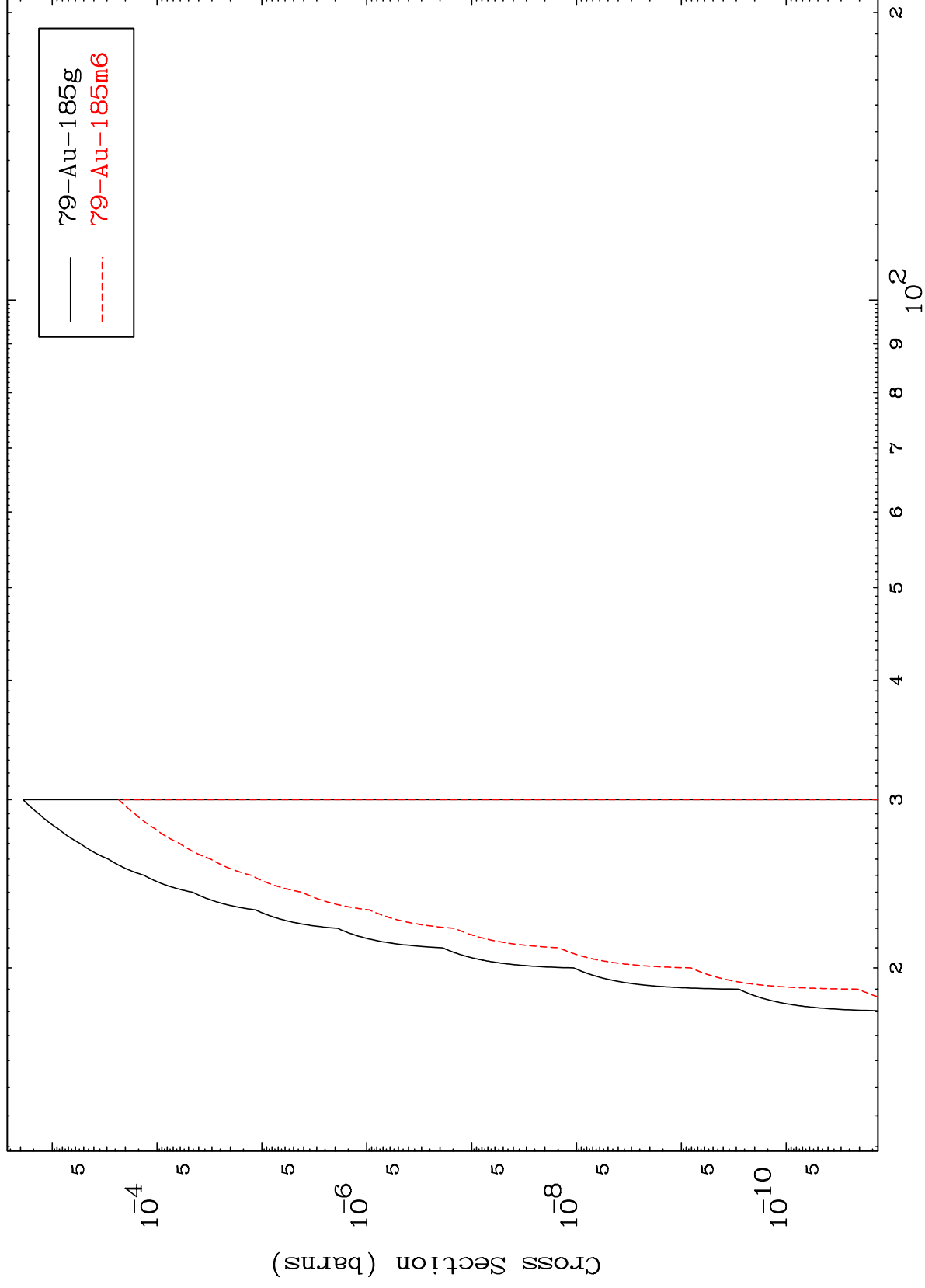
80-Hg-188

MAT 8001

(n,n') t

80-Hg-188

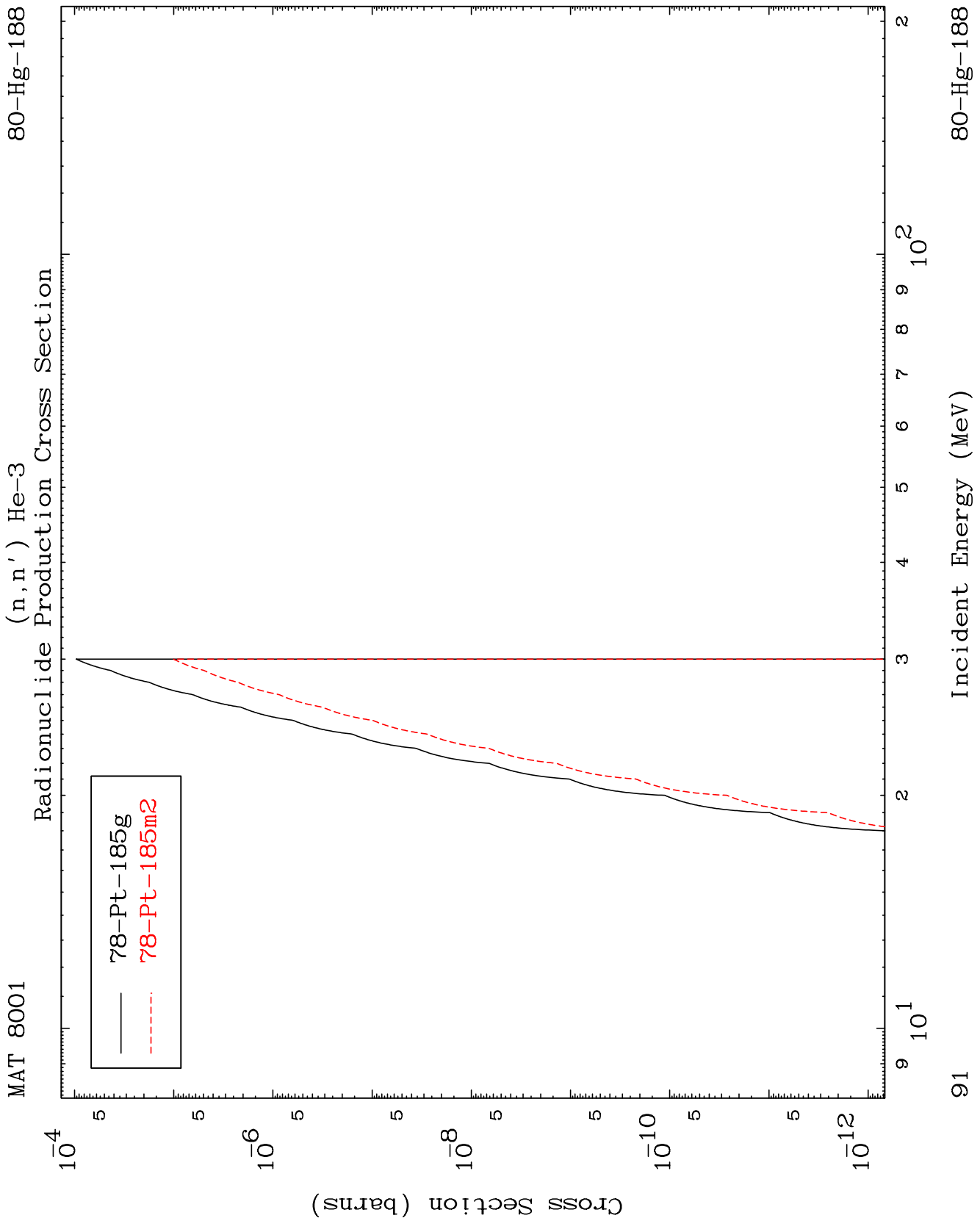
Radionuclide Production Cross Section

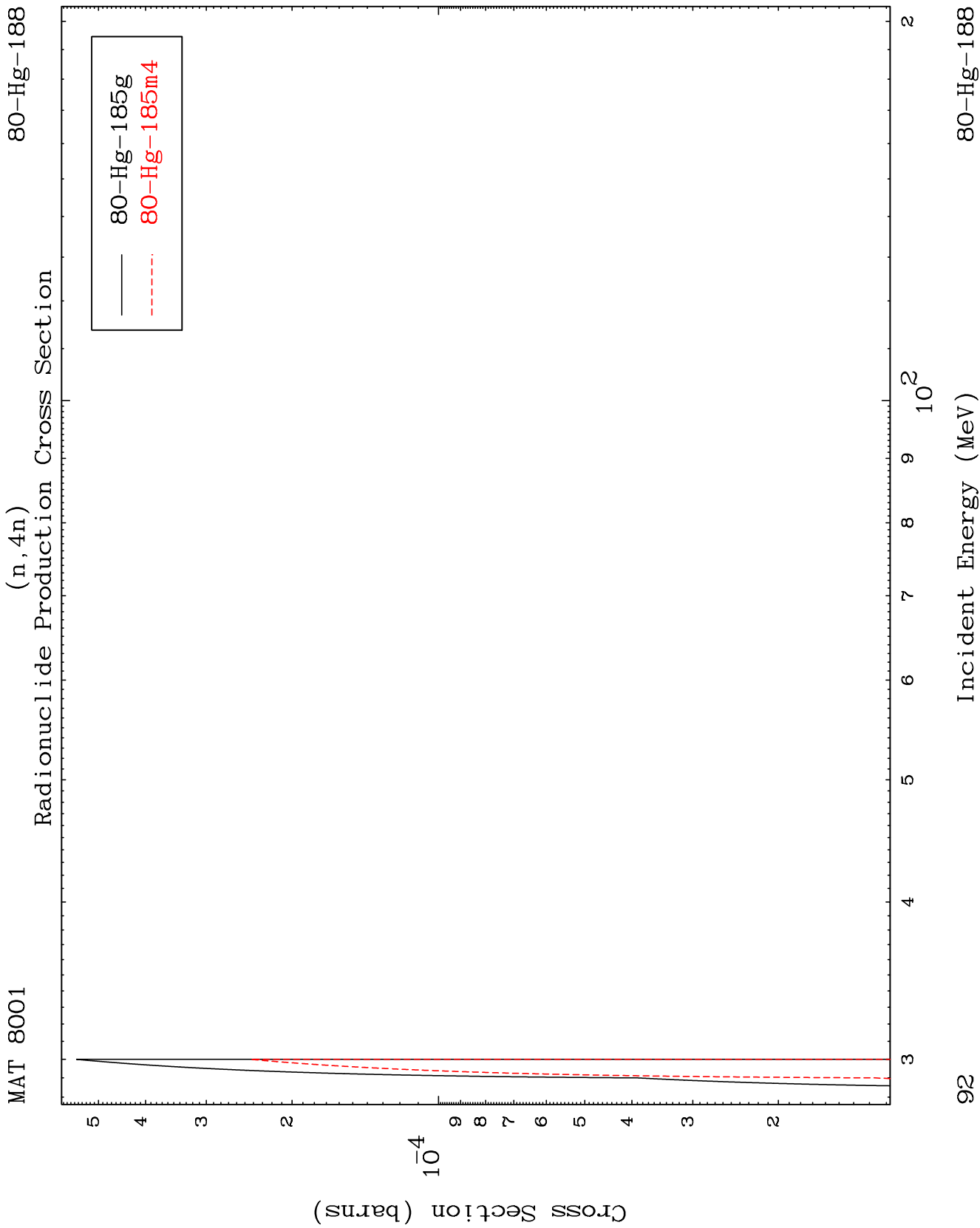


90

Incident Energy (MeV)

80-Hg-188



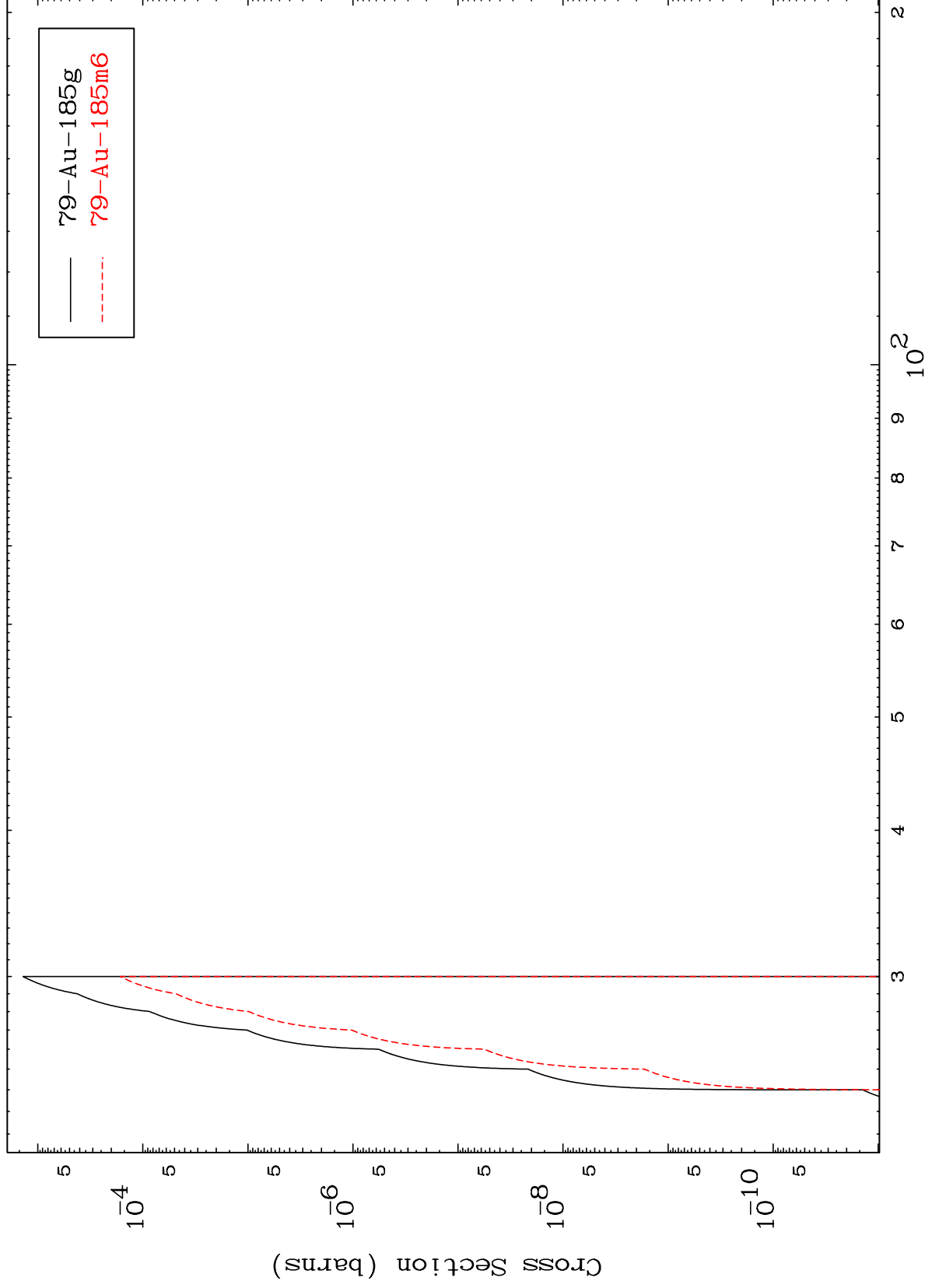


MAT 8001

(n,3n) p

80-Hg-188

Radionuclide Production Cross Section



93

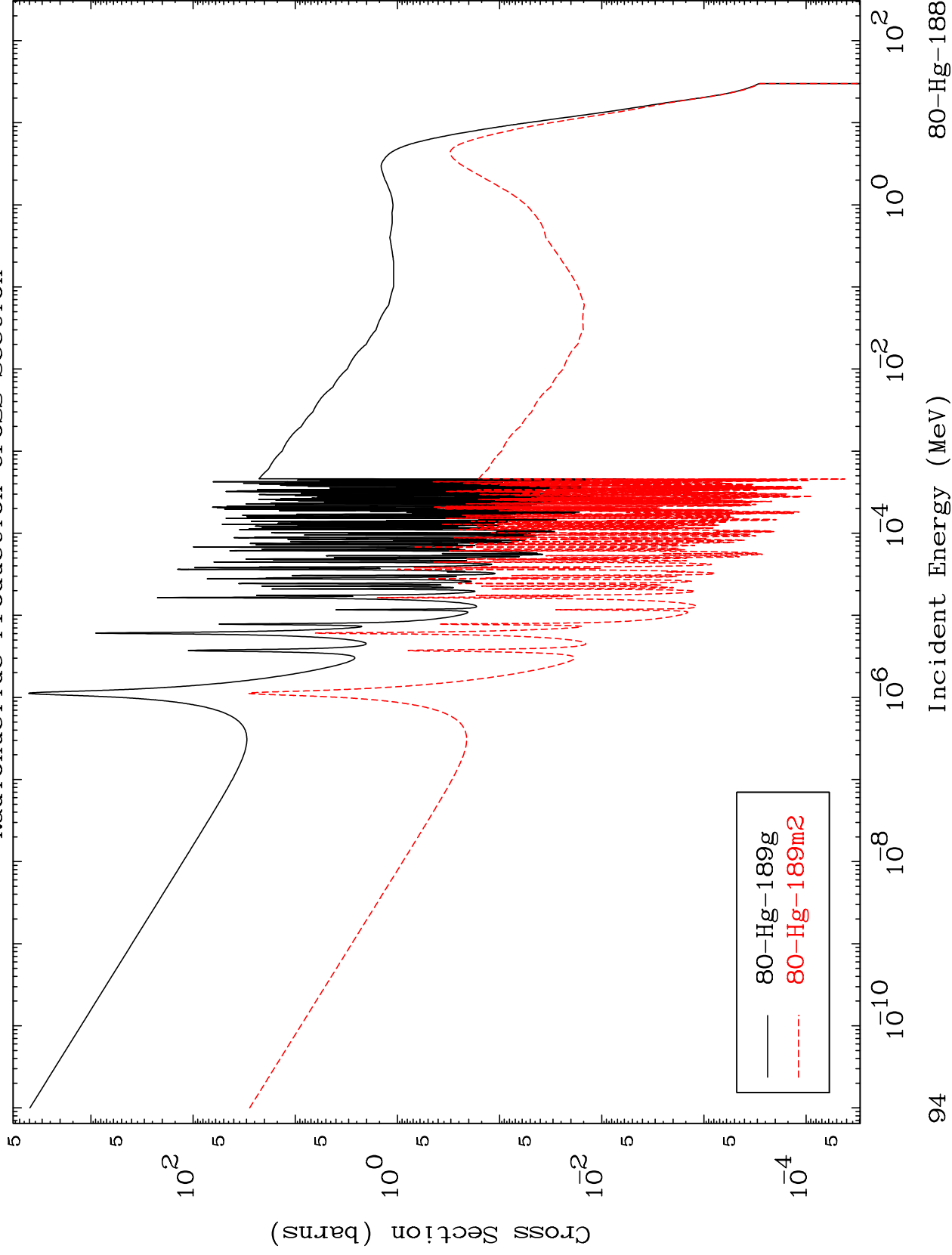
Incident Energy (MeV)

80-Hg-188

MAT 8001

80-Hg-188

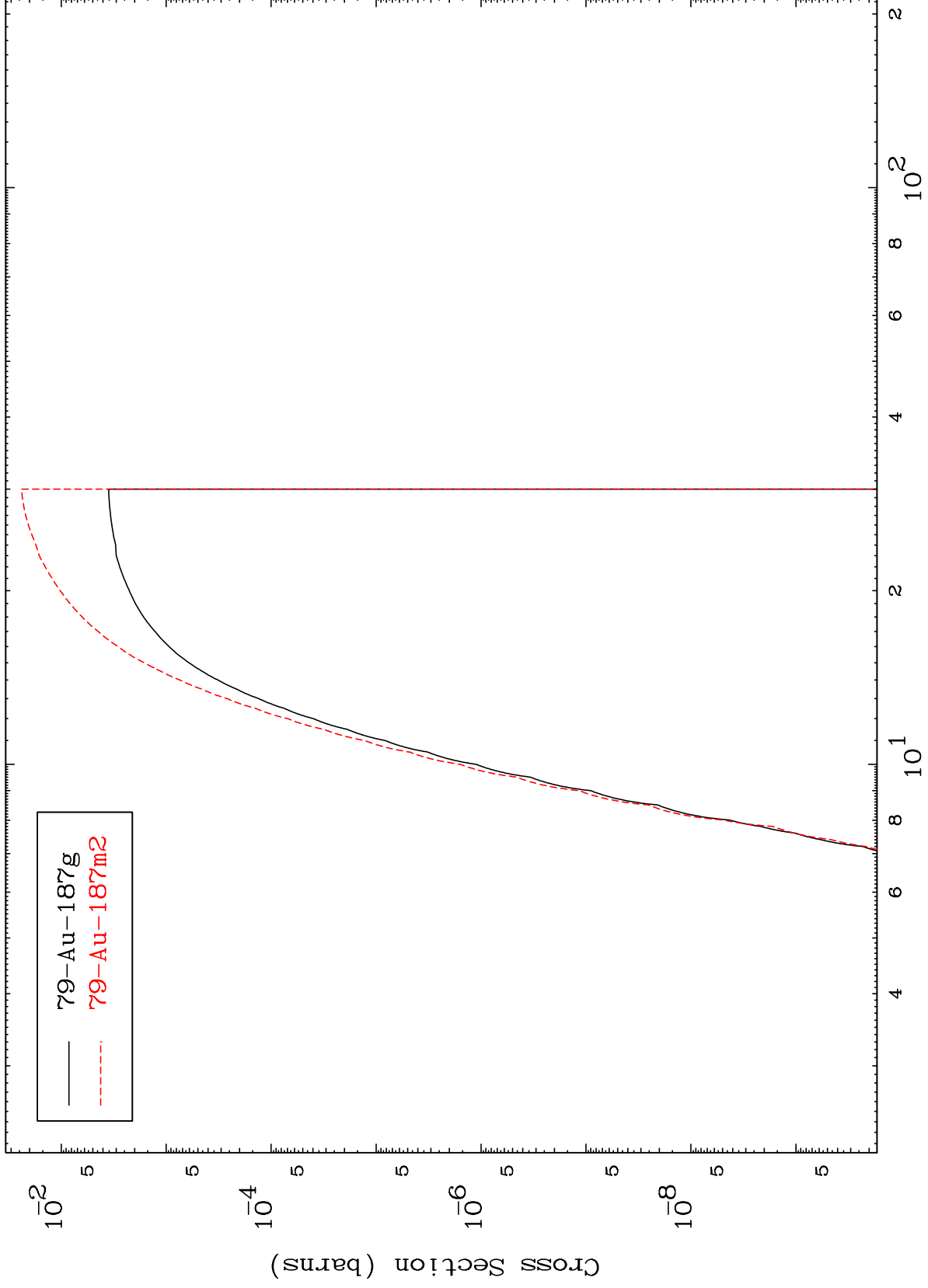
Radionuclide Production Cross Section  
(n,  $\gamma$ )



MAT 8001

80-Hg-188

(n,d)  
Radionuclide Production Cross Section



95

80-Hg-188

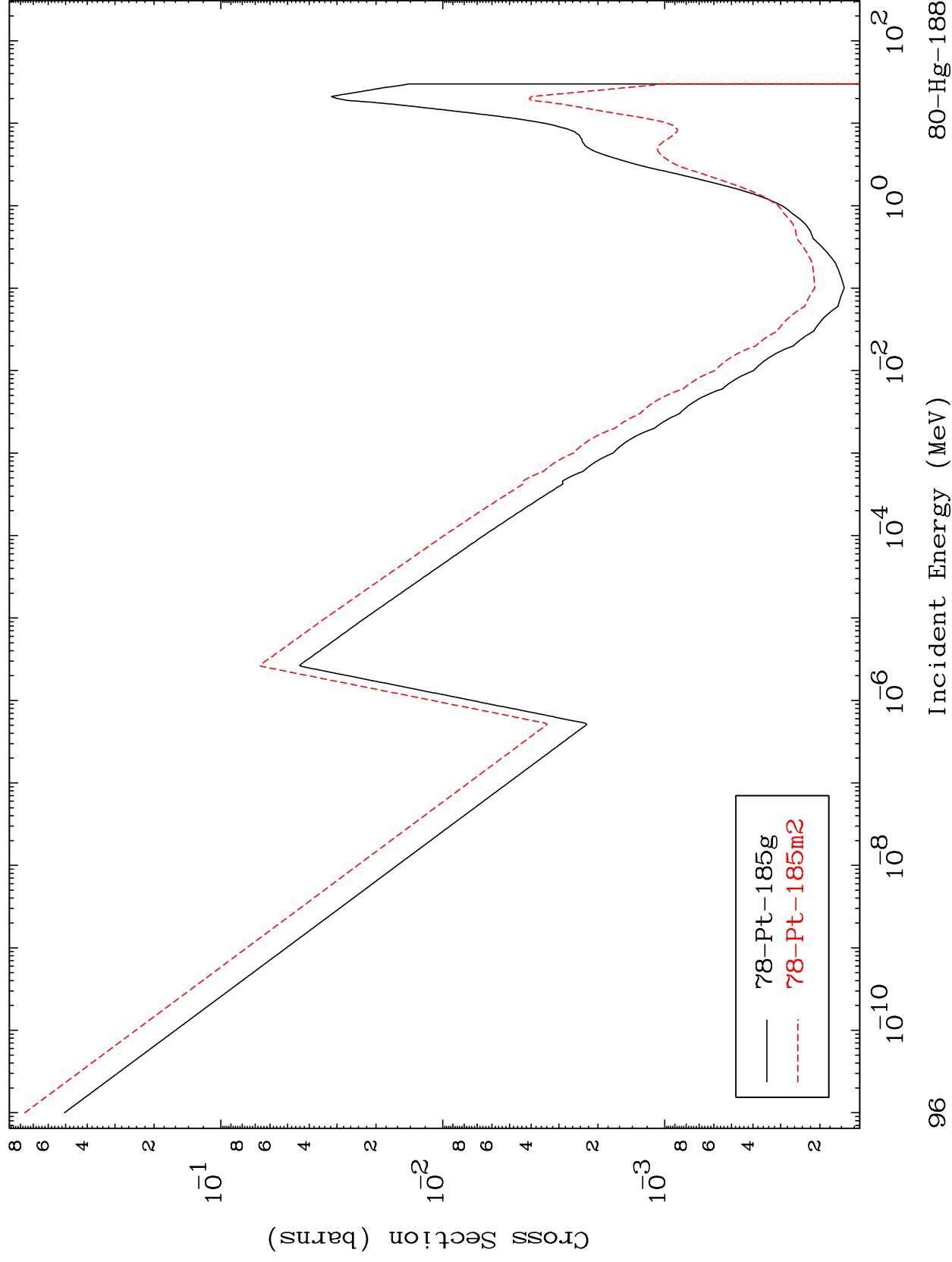
Incident Energy (MeV)



MAT 8001

80-Hg-188

$(n, \alpha)$   
Radionuclide Production Cross Section



96

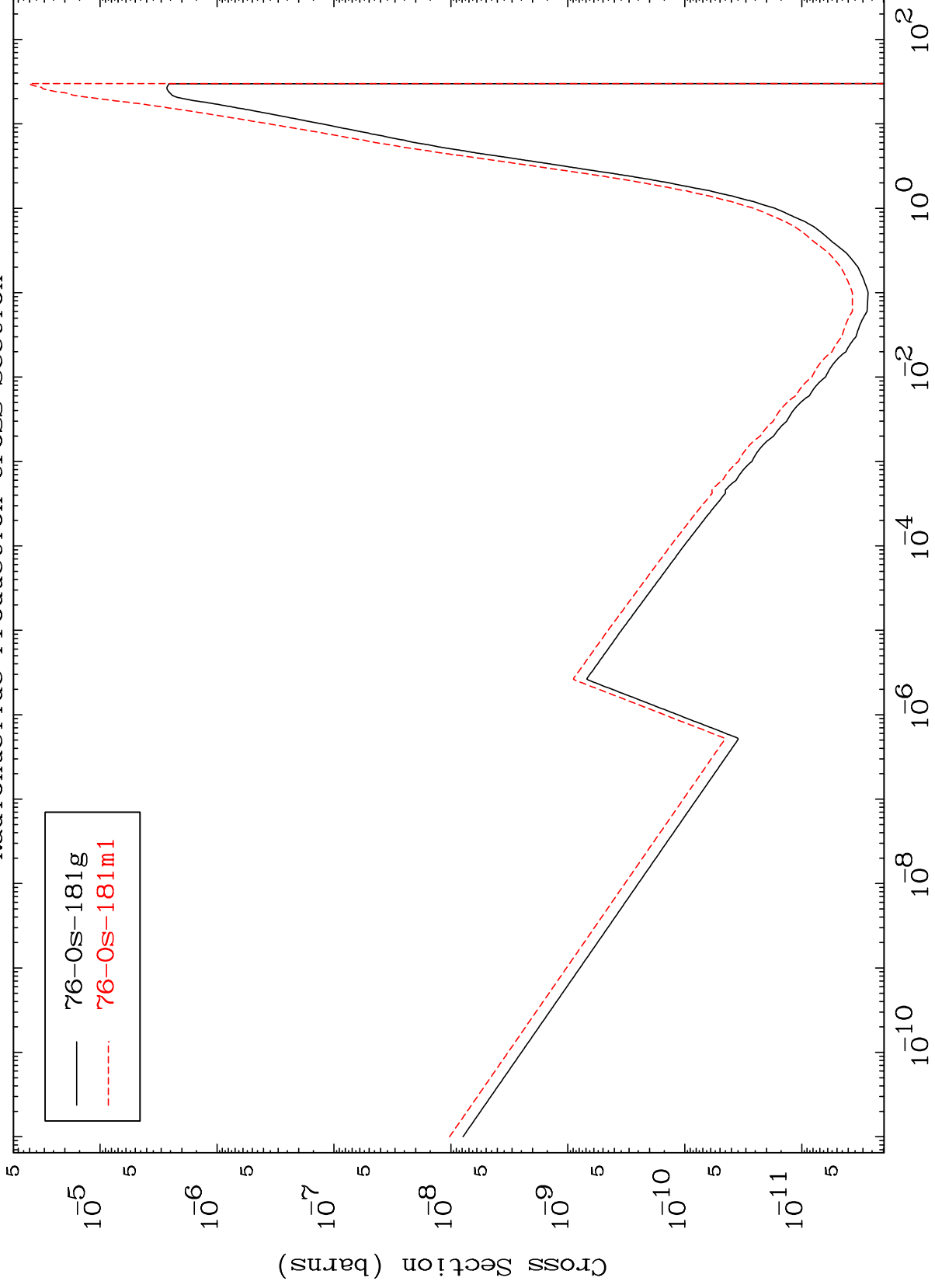
80-Hg-188

MAT 8001

(n,2α)

80-Hg-188

Radionuclide Production Cross Section

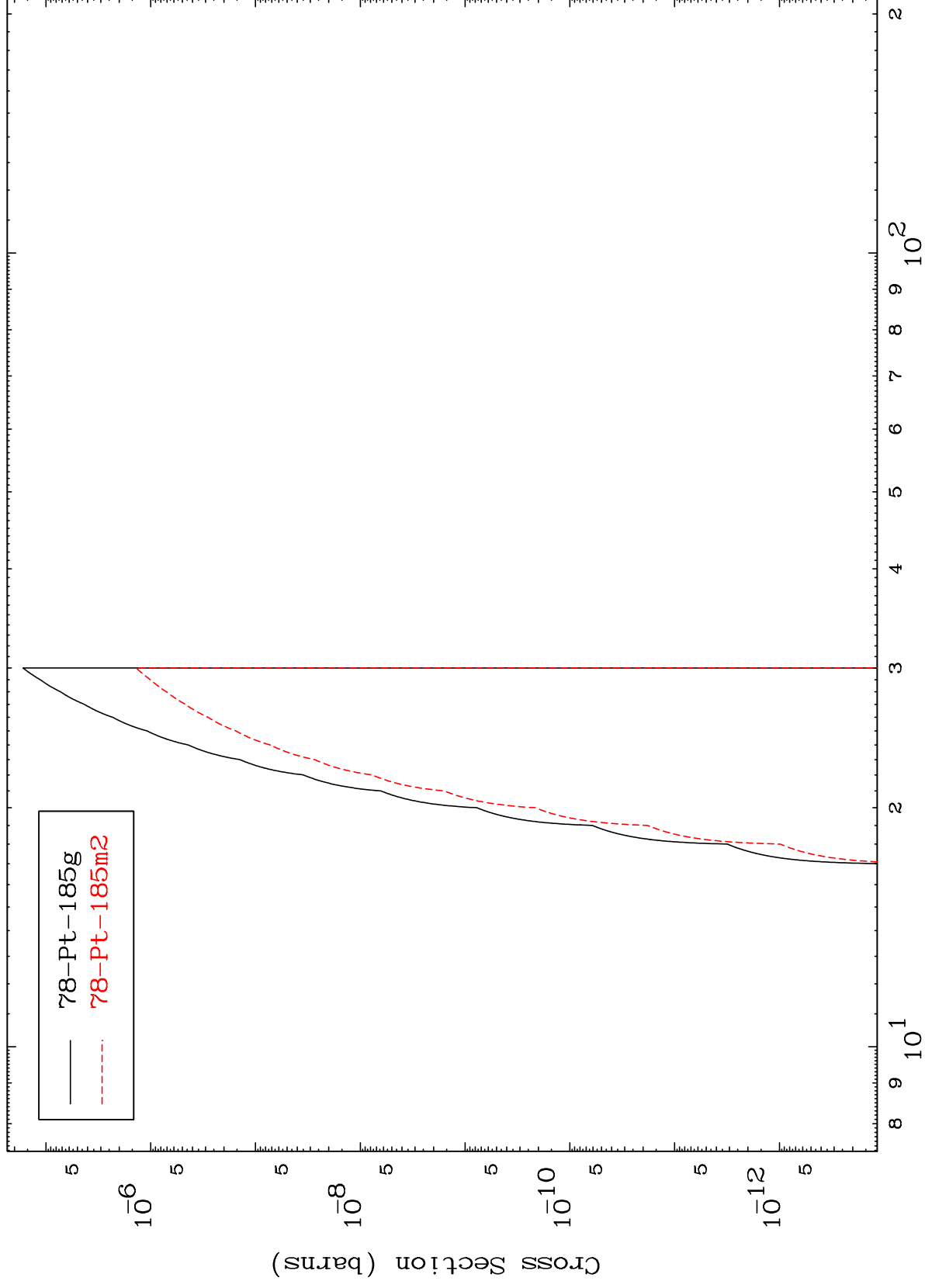


MAT 8001

80-Hg-188

(n,p) t

Radionuclide Production Cross Section



98

Incident Energy (MeV)

80-Hg-188