

Program EVALPLOT  
(Version 2015-2)

by

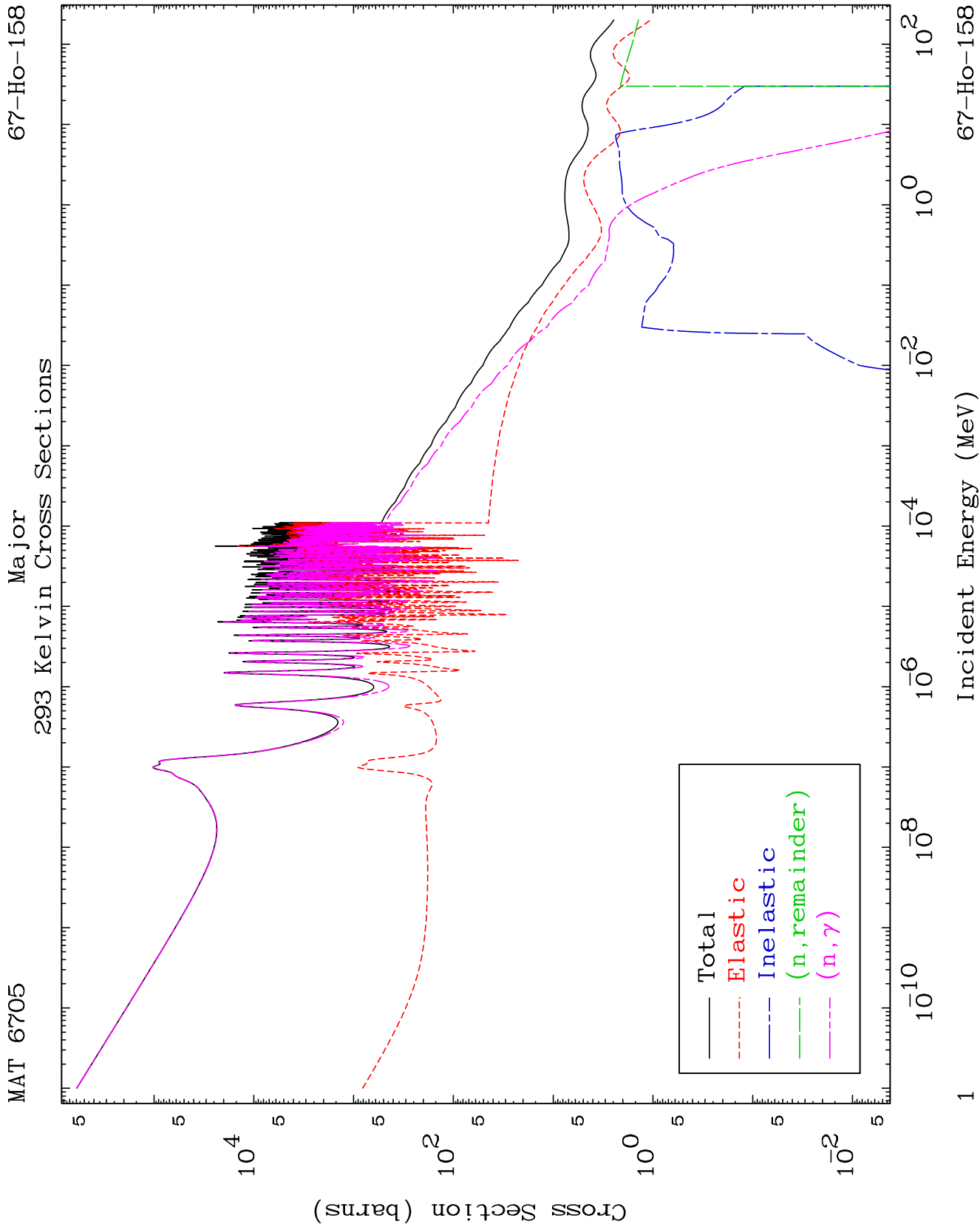
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(Present Contact Information)

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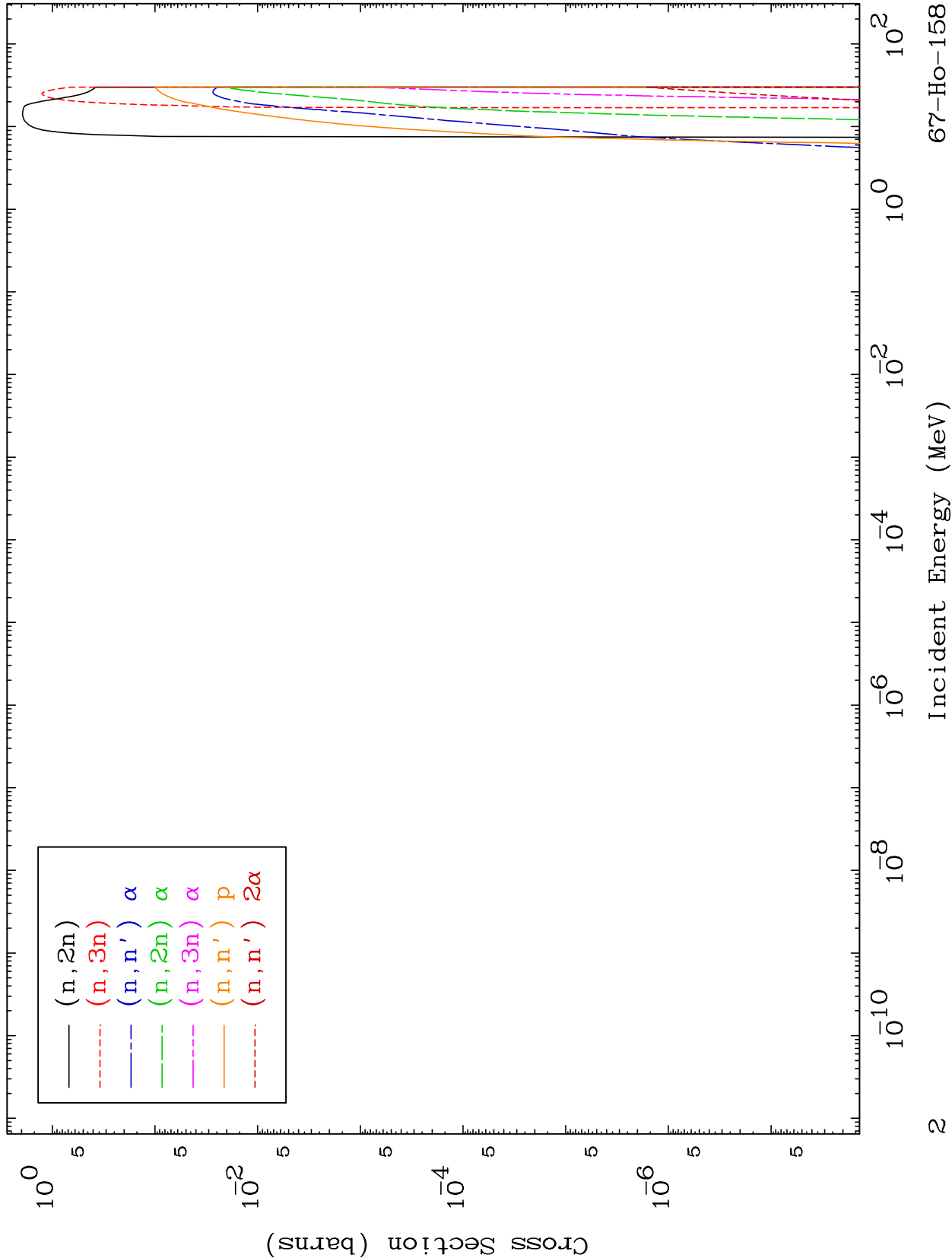
Press Mouse Button to Start

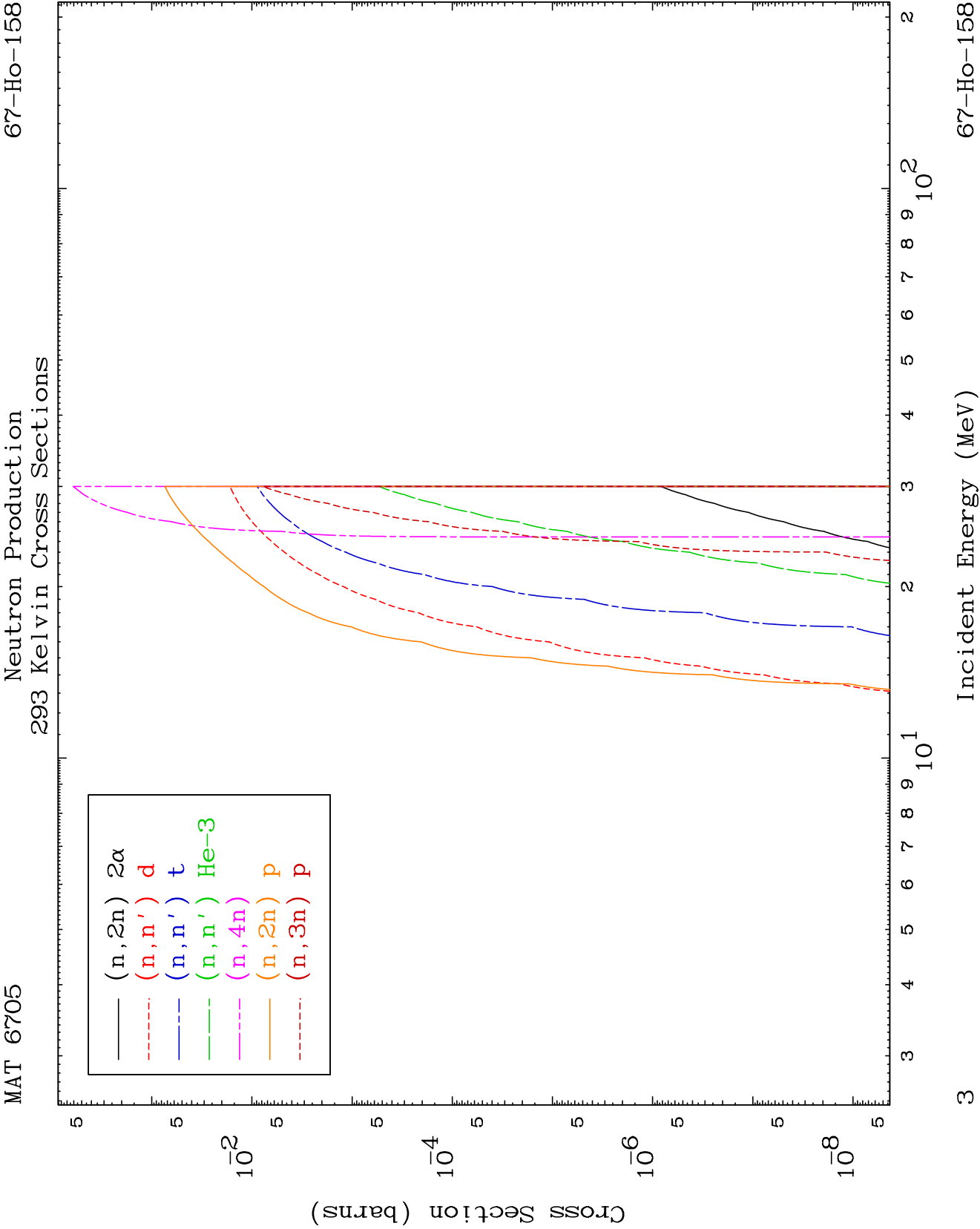


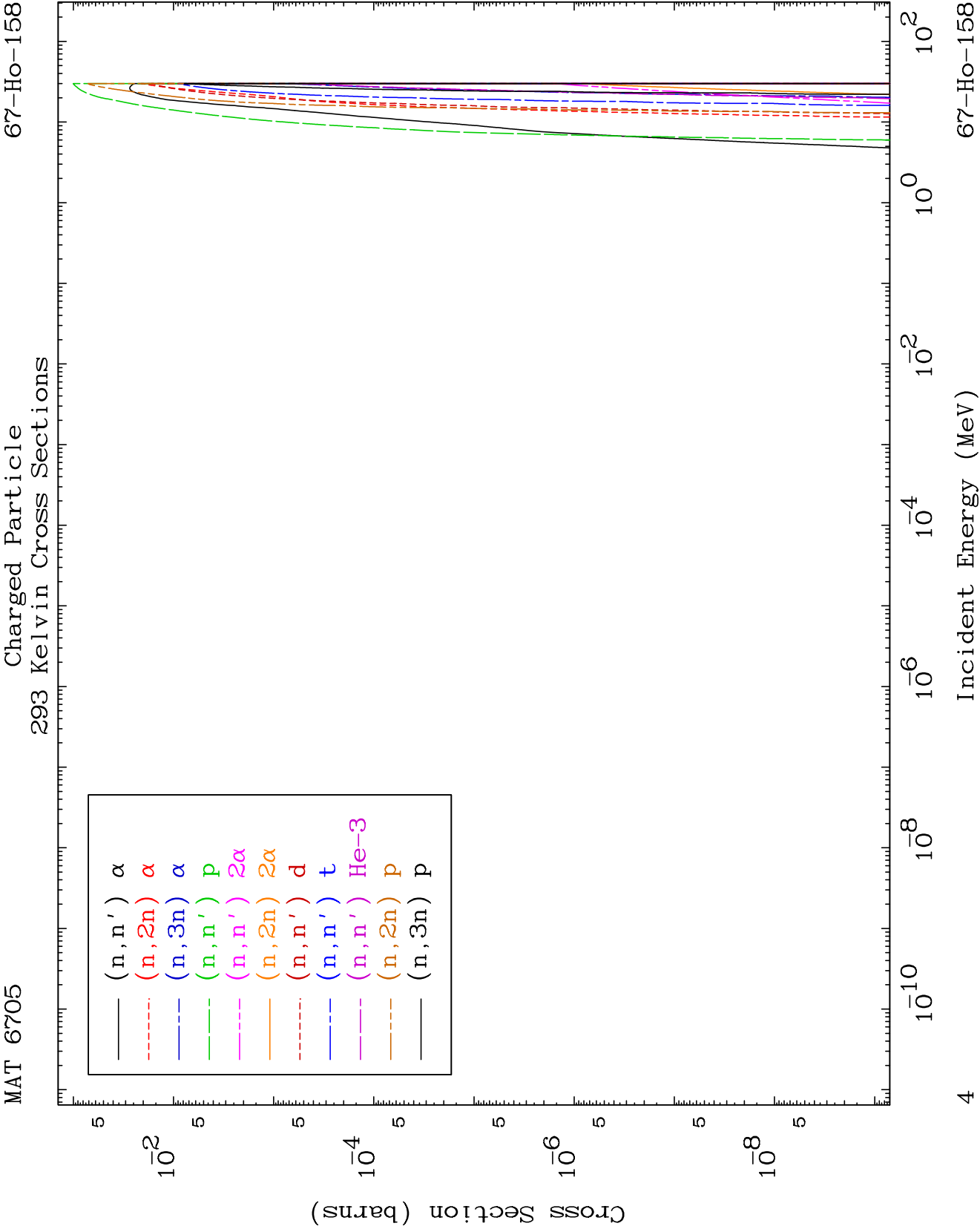
MAT 6705

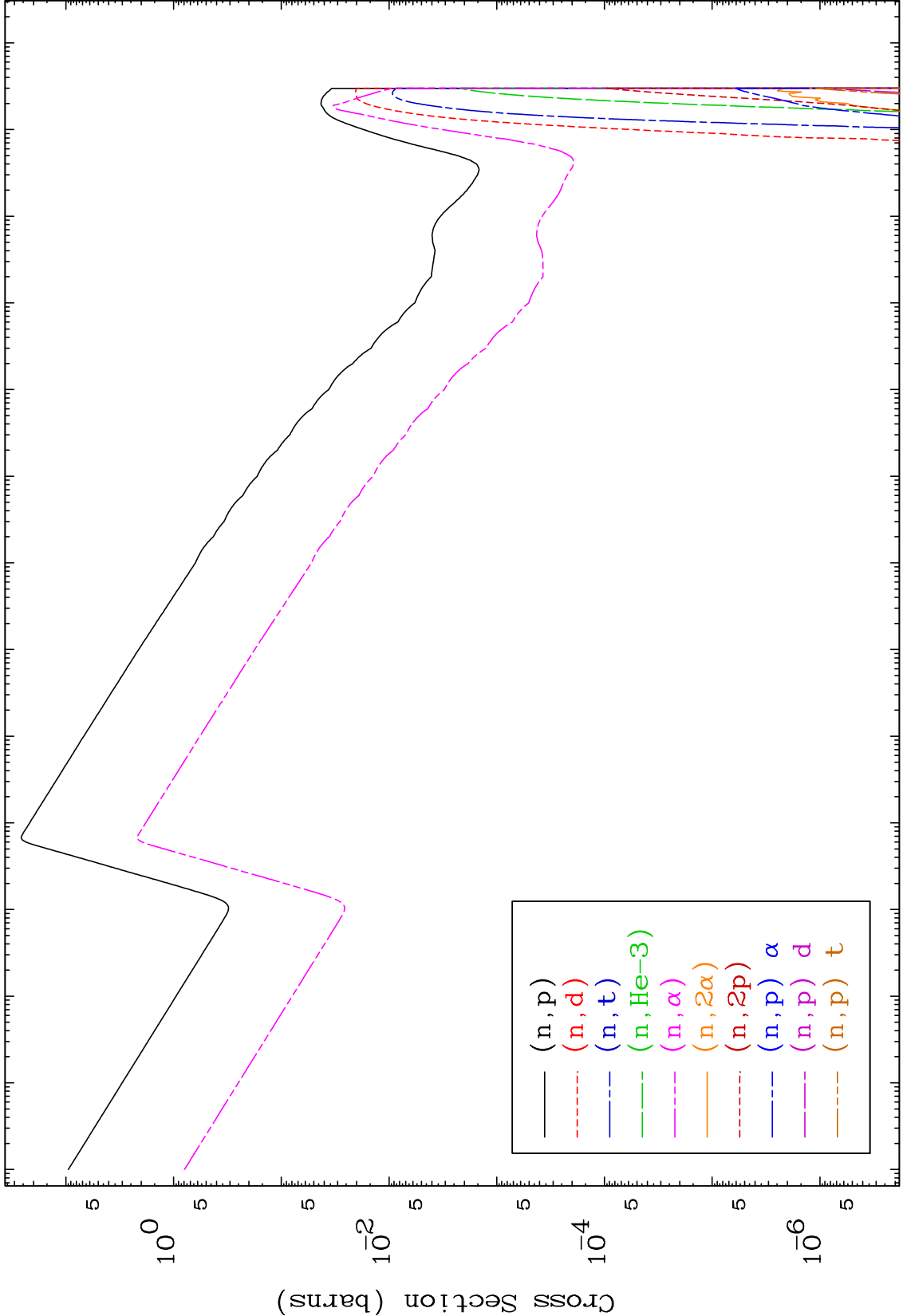
Neutron Production  
293 Kelvin Cross Sections

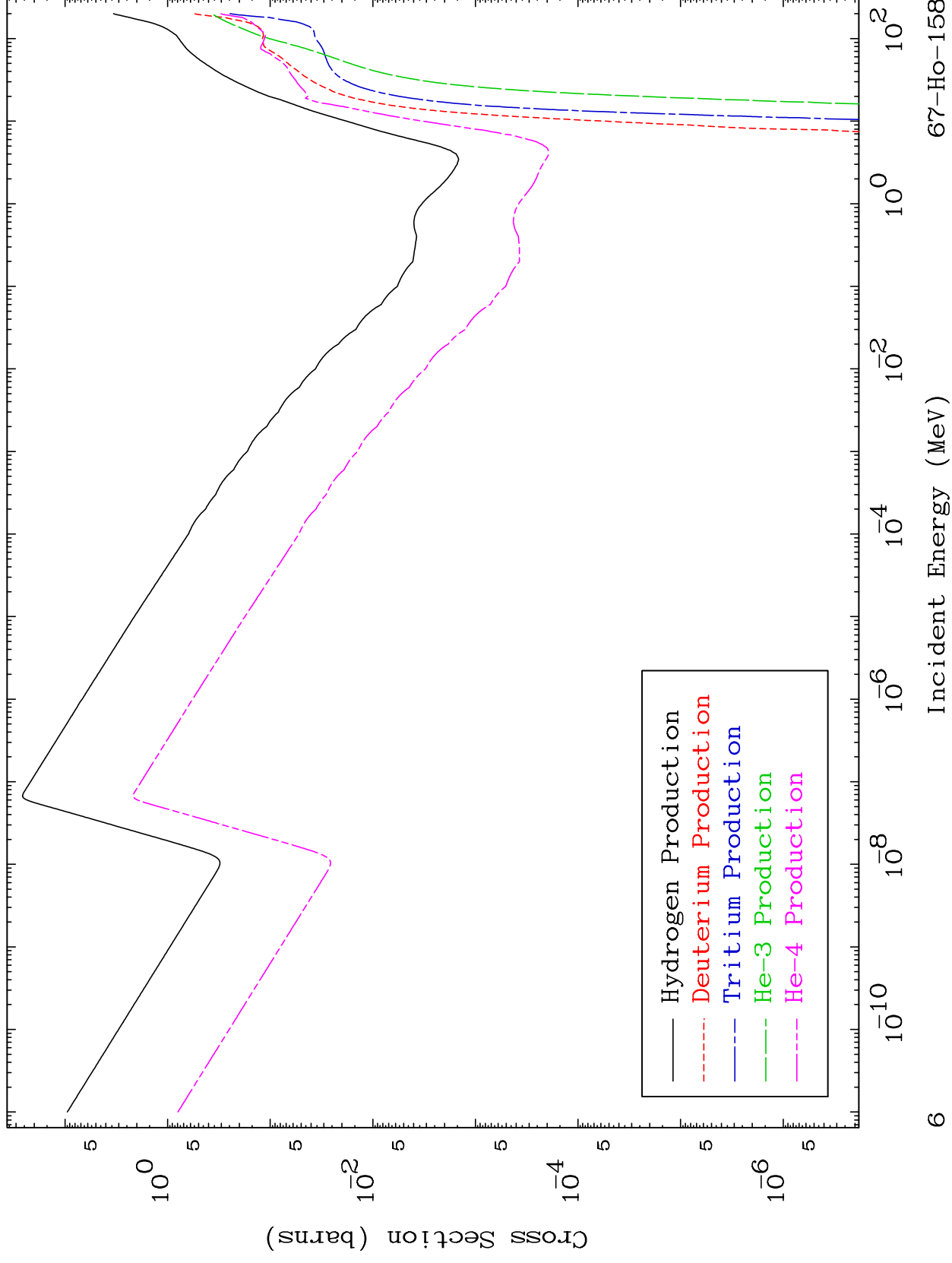
67-Ho-158

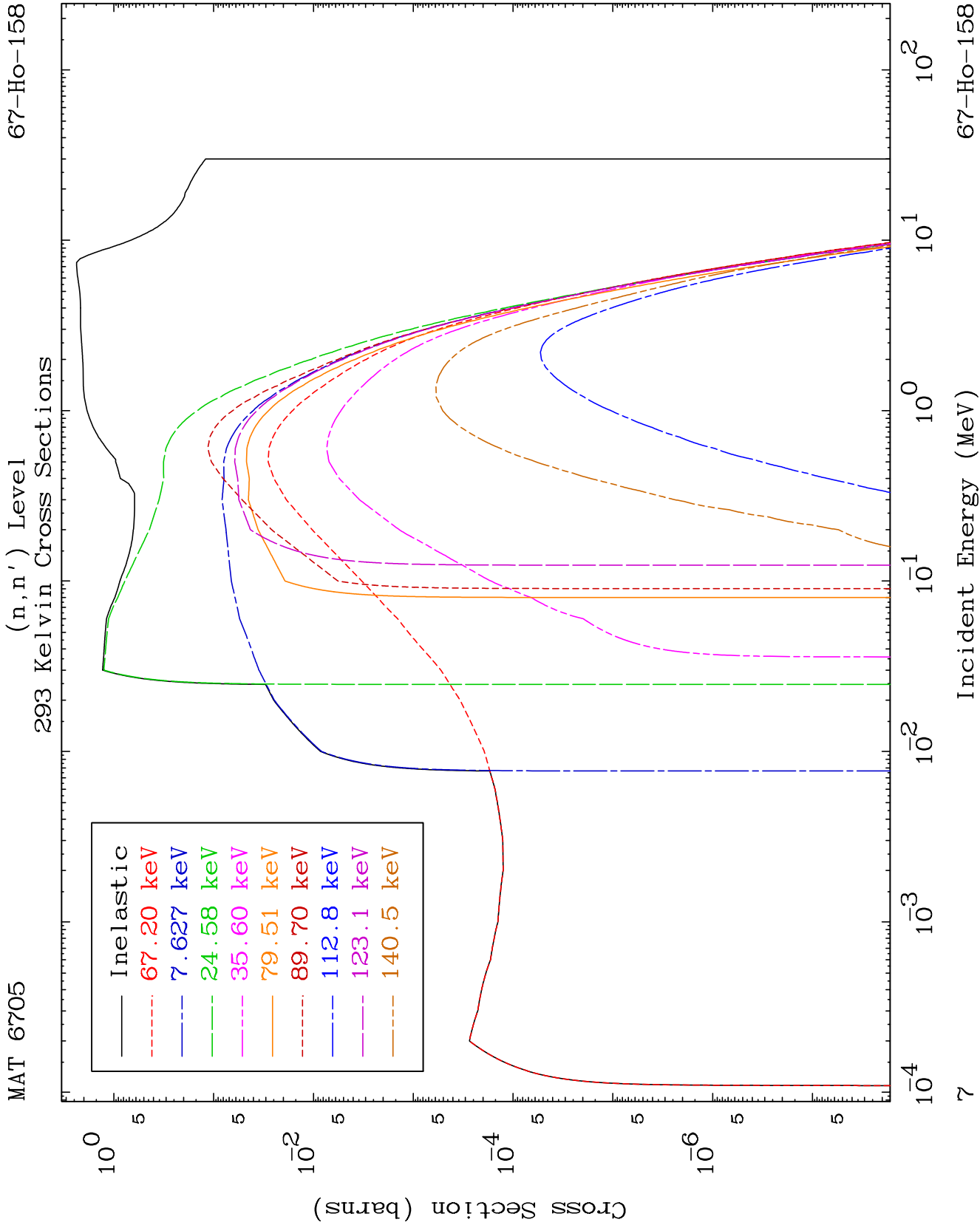




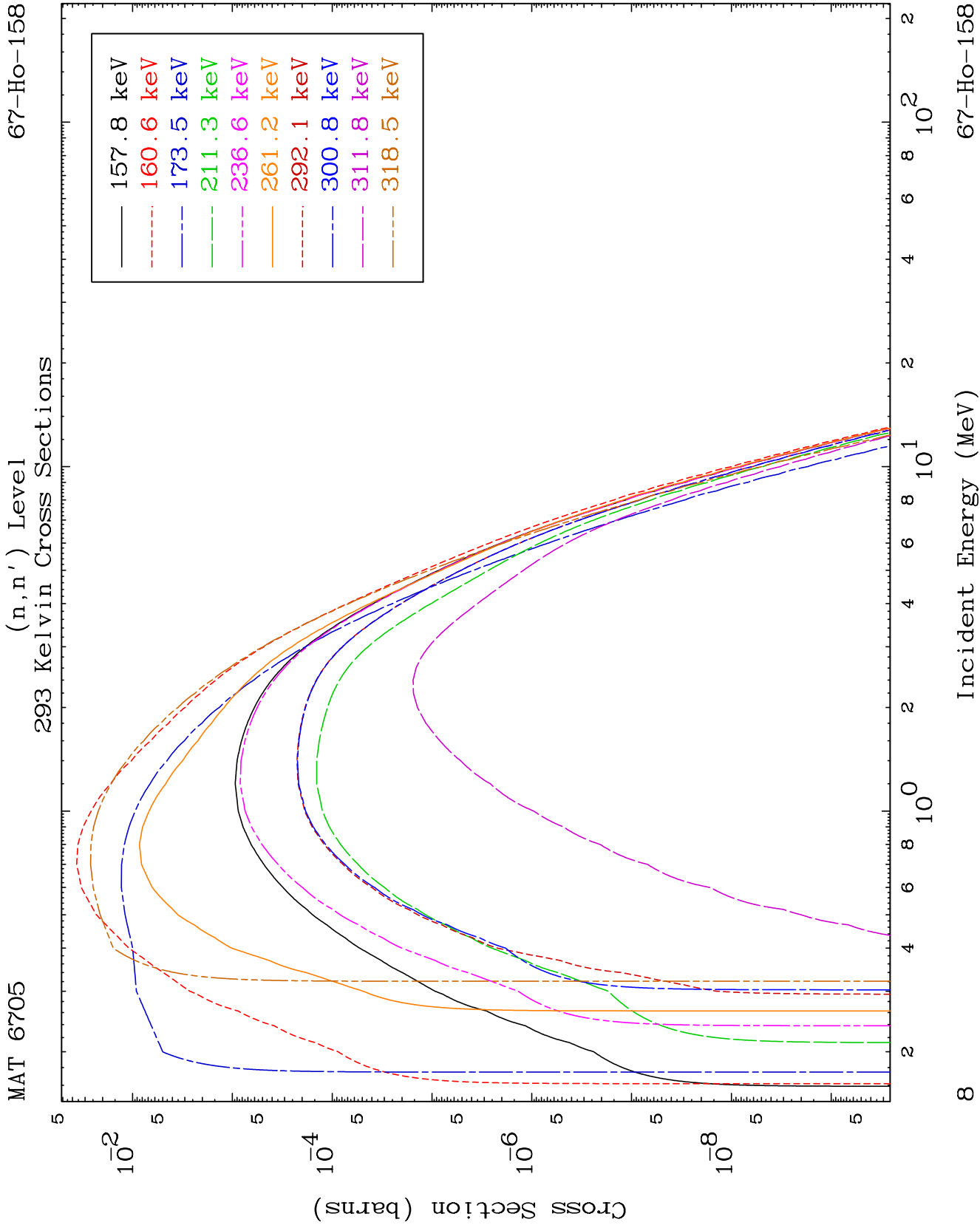


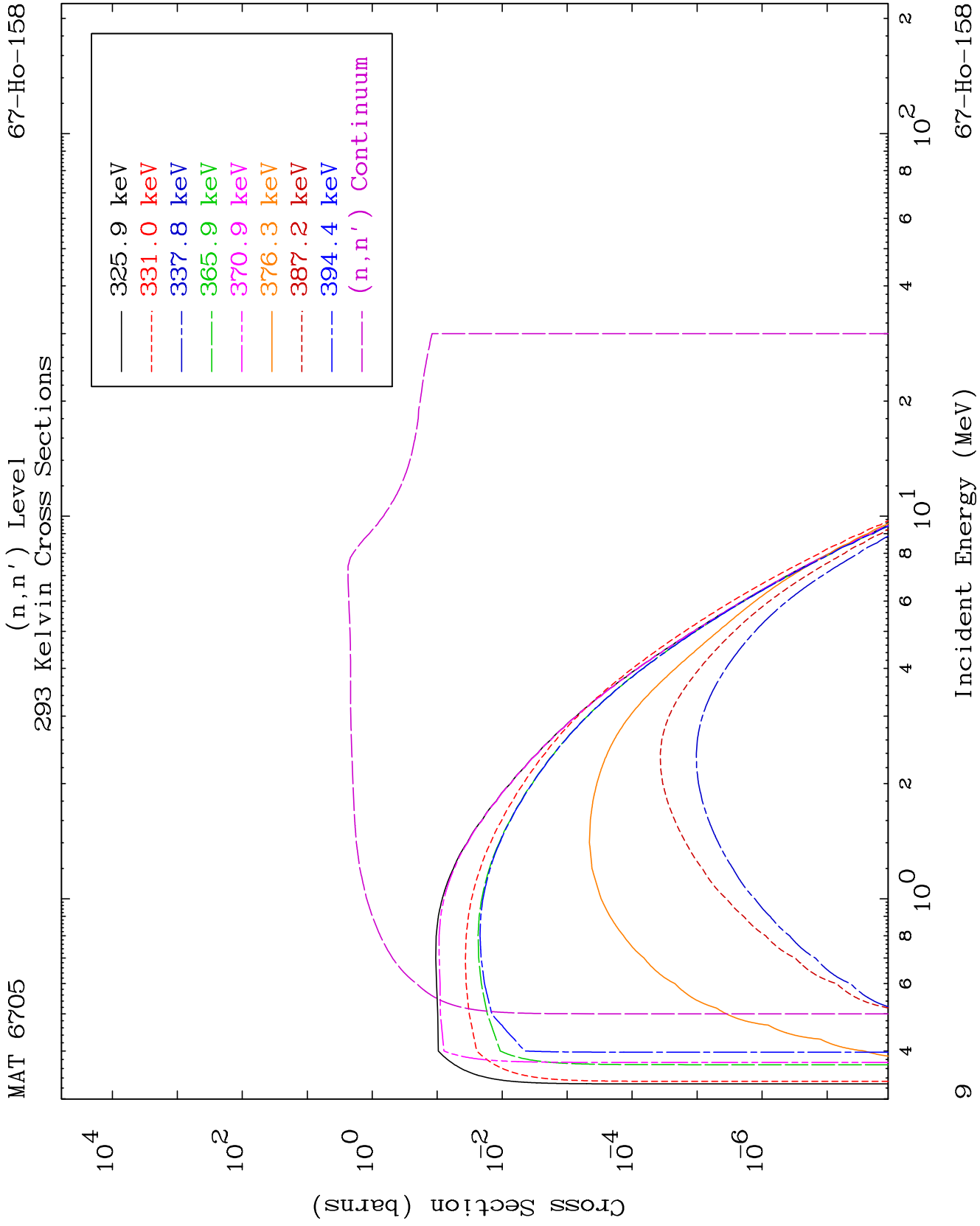


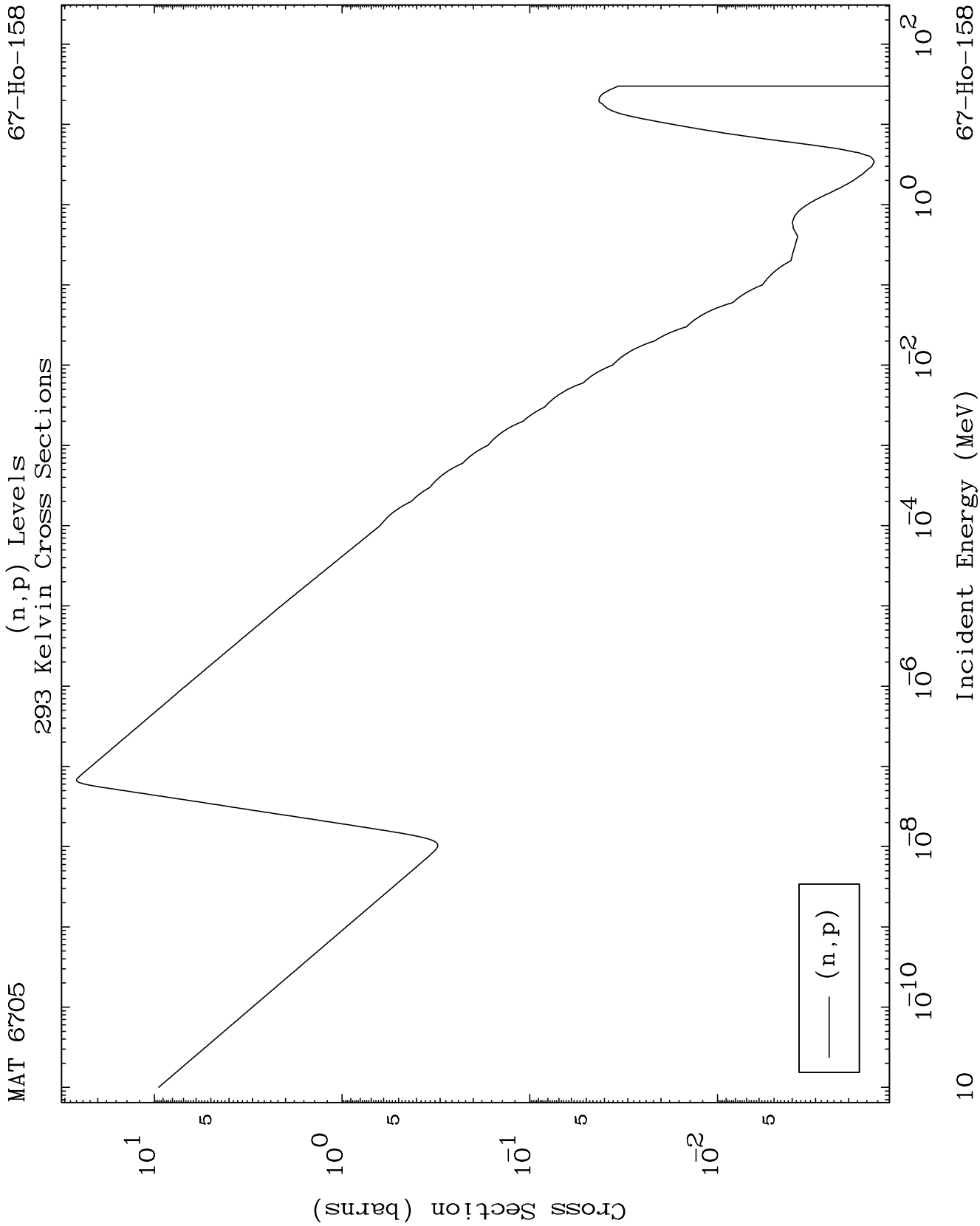








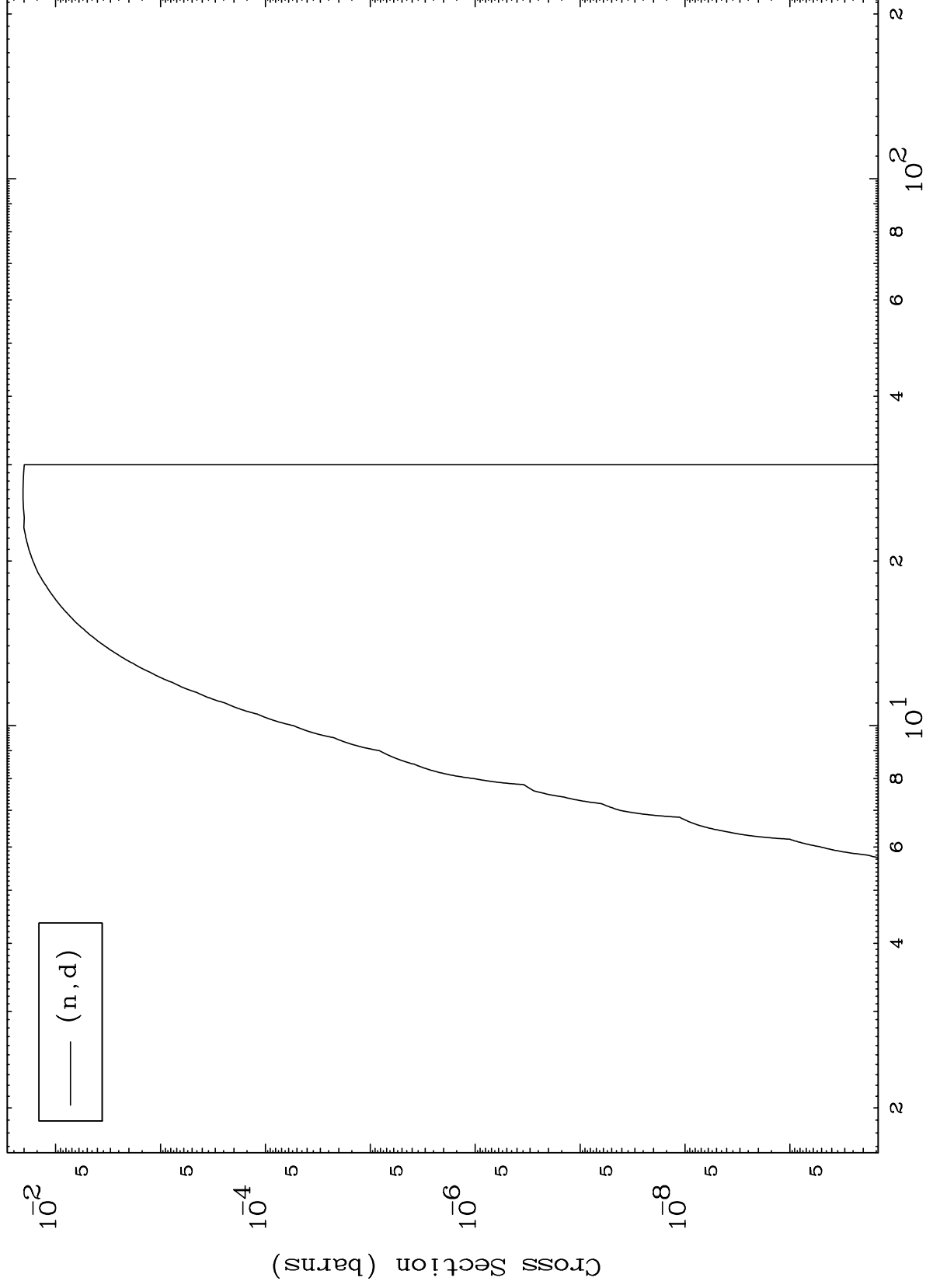




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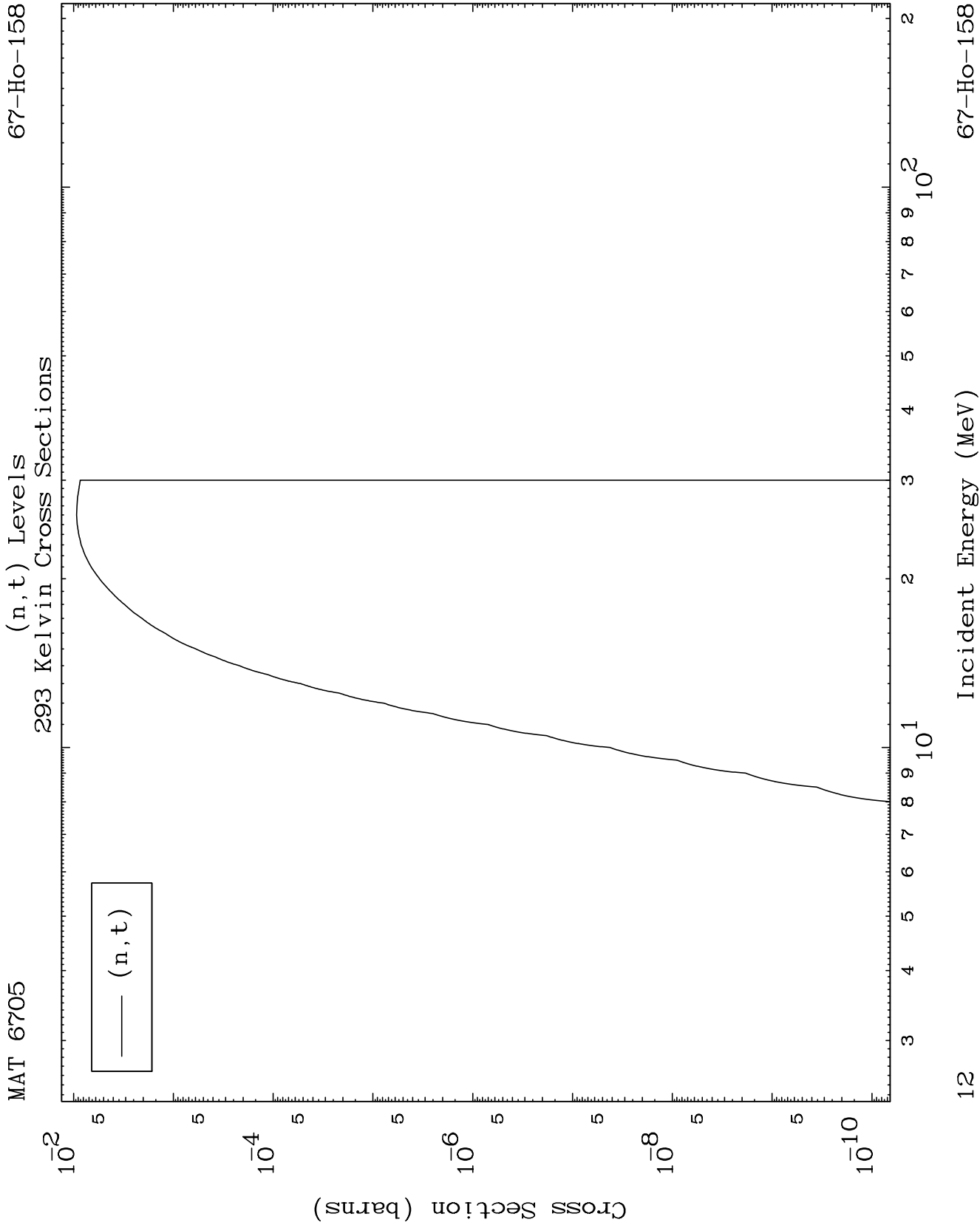
(n,d) Levels  
293 Kelvin Cross Sections



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Incident Energy (MeV)

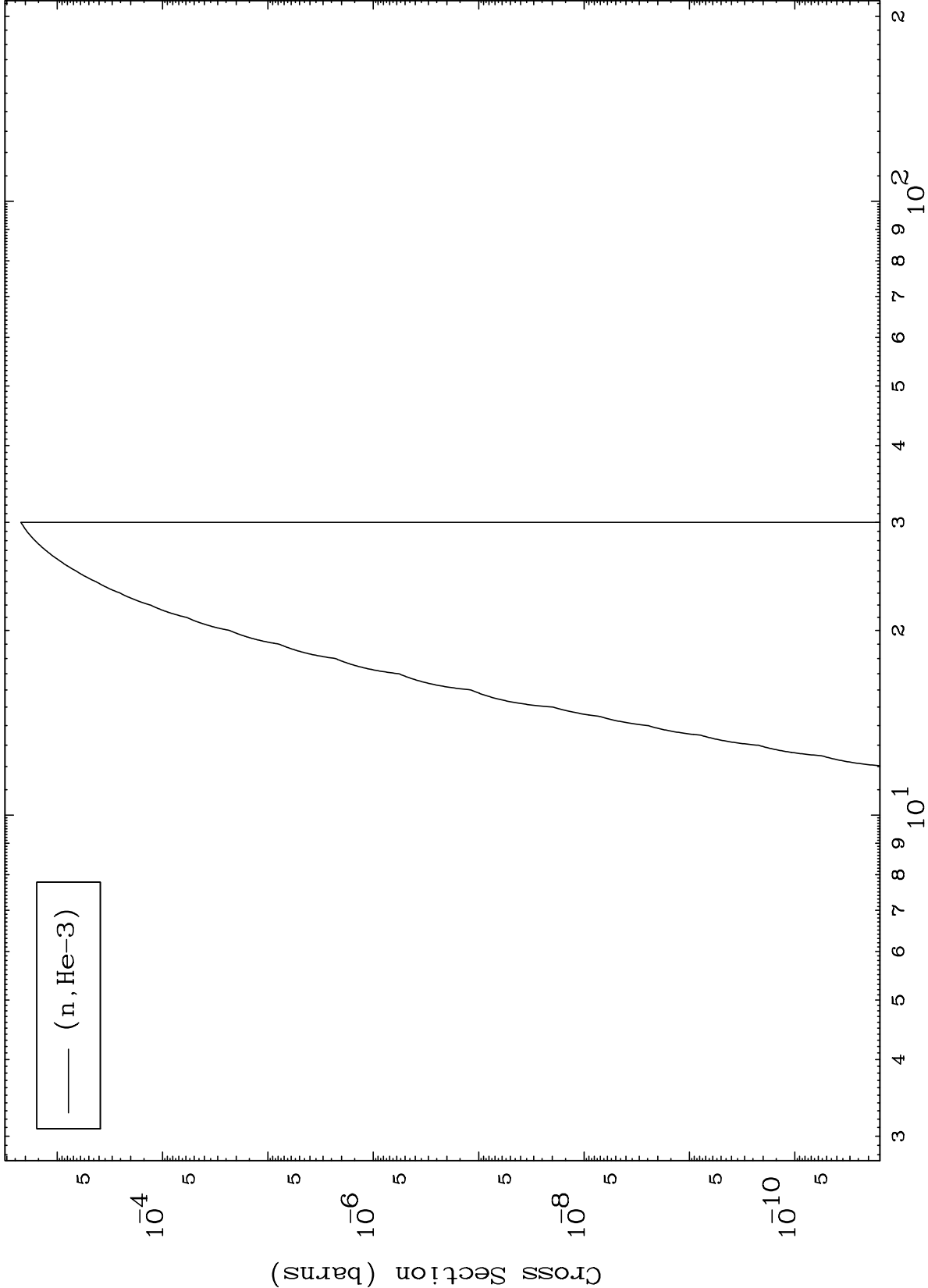
67-Ho-158



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(n,He3) Levels  
293 Kelvin Cross Sections

67-Ho-158



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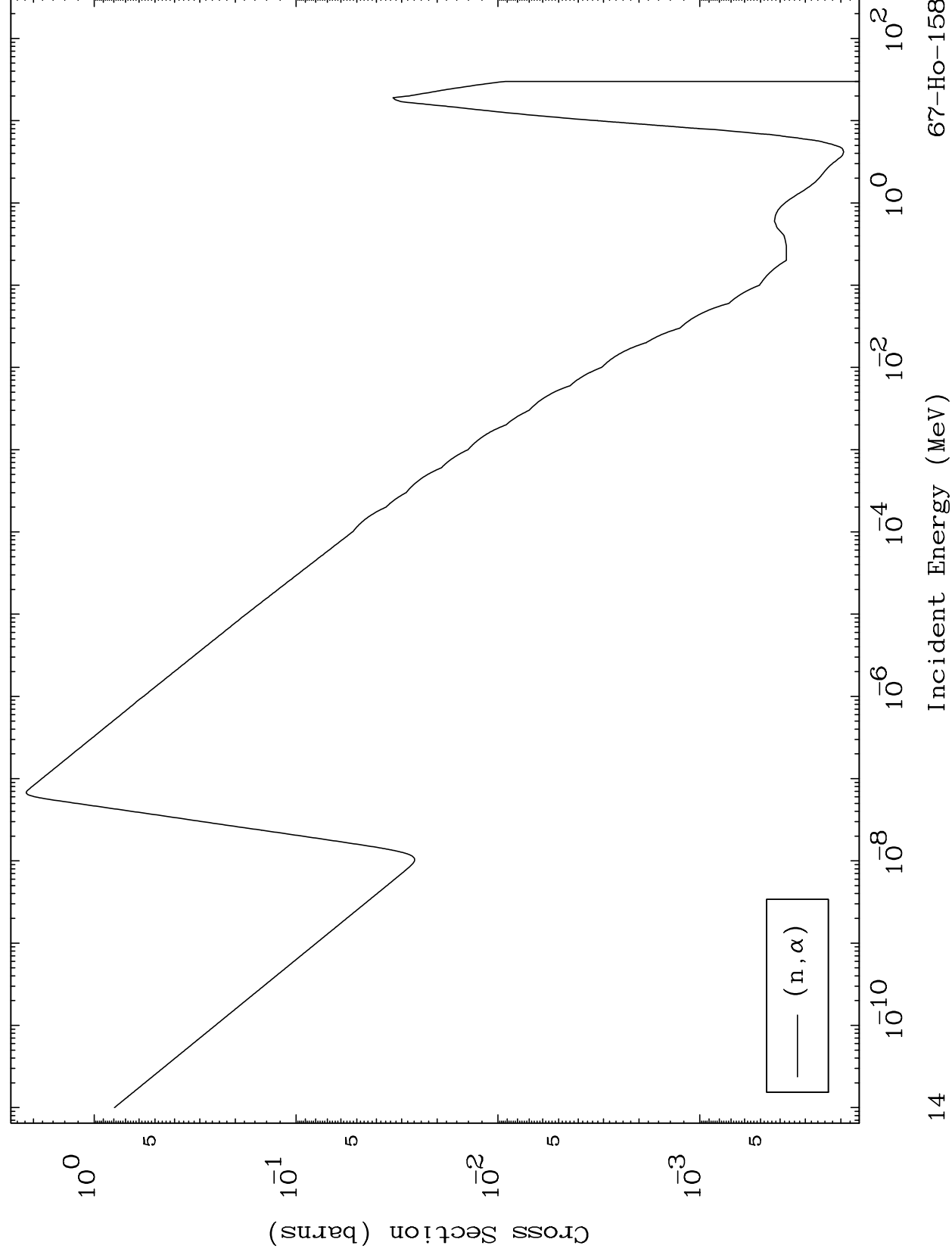
Incident Energy (MeV)

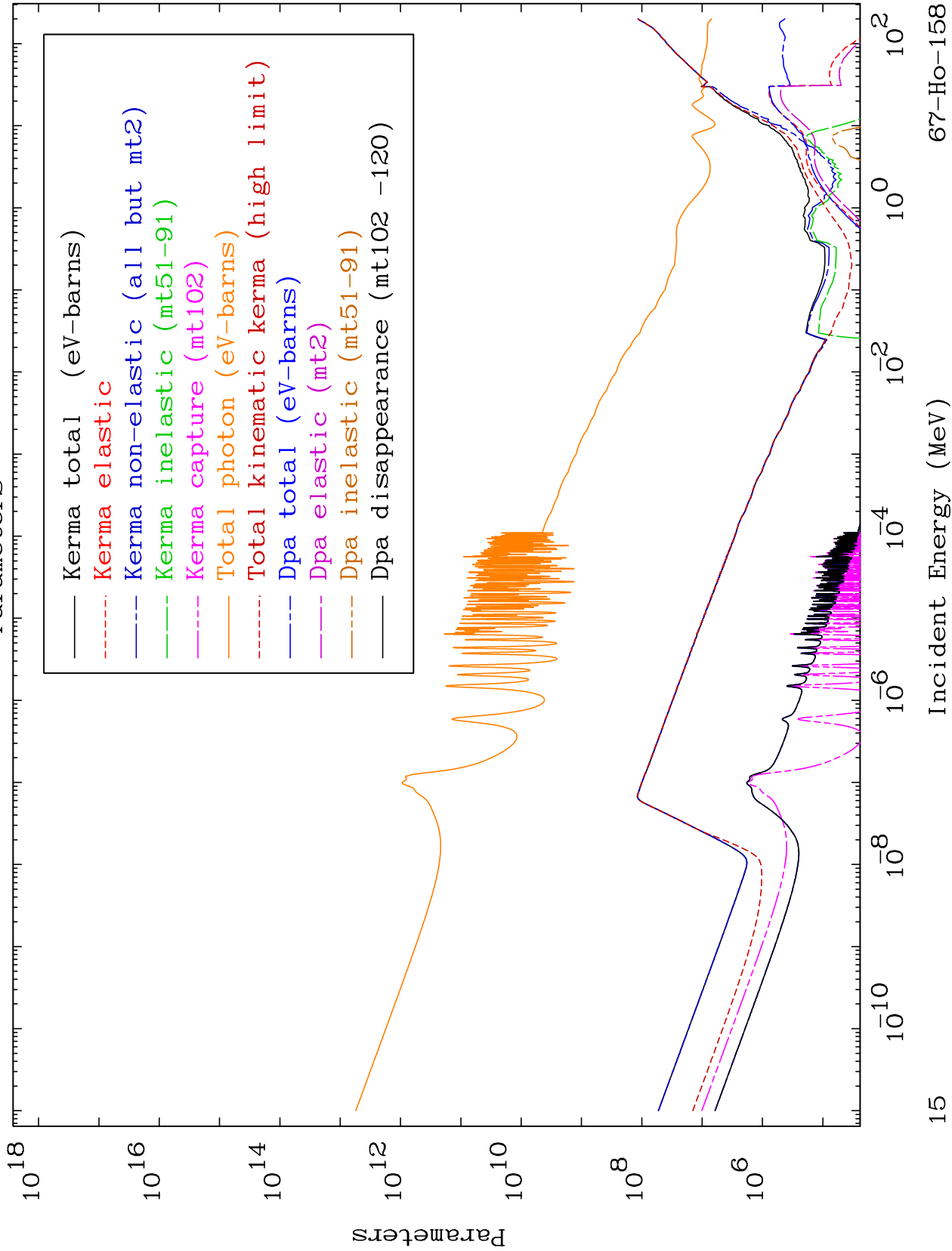
67-Ho-158

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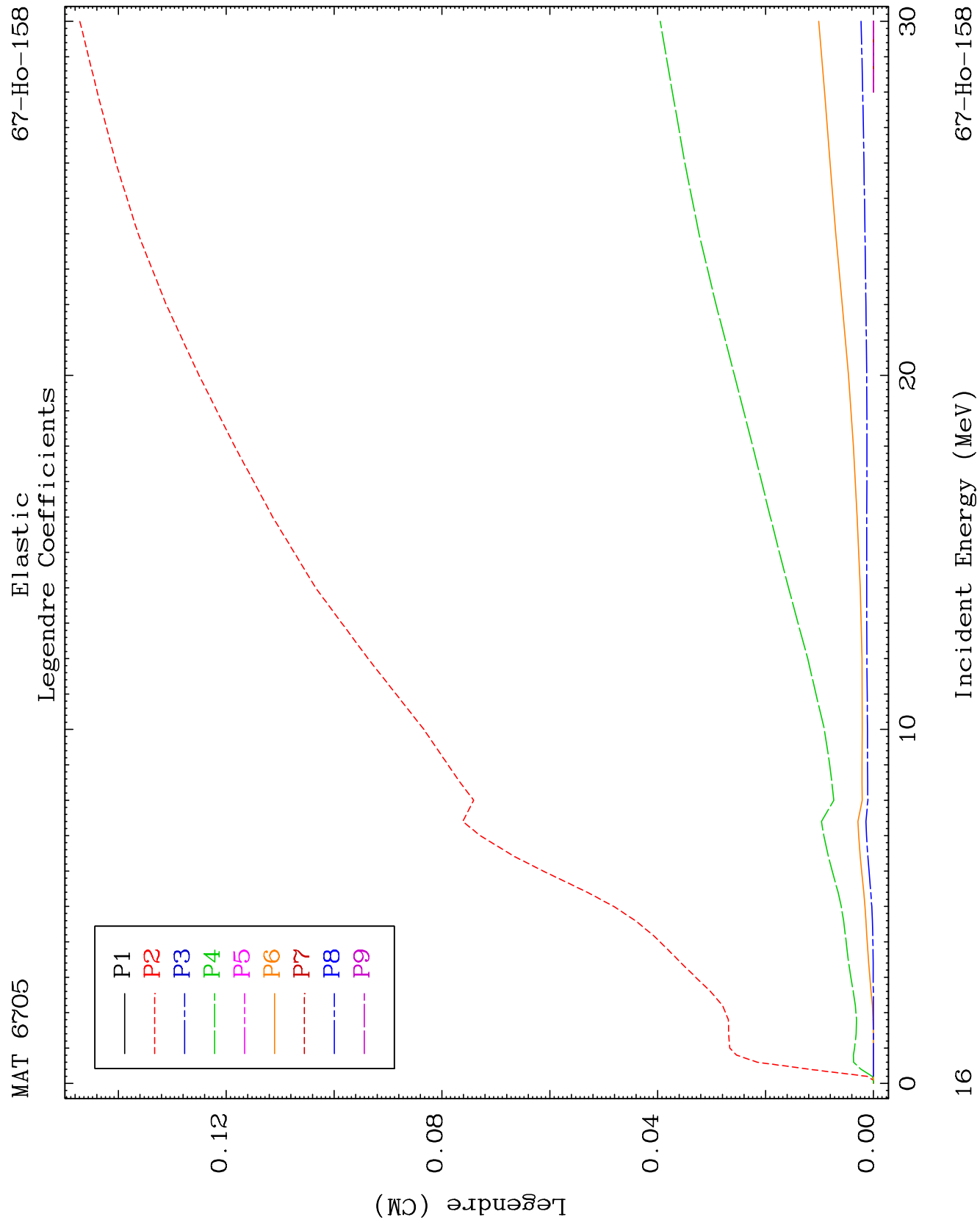
67-Ho-158

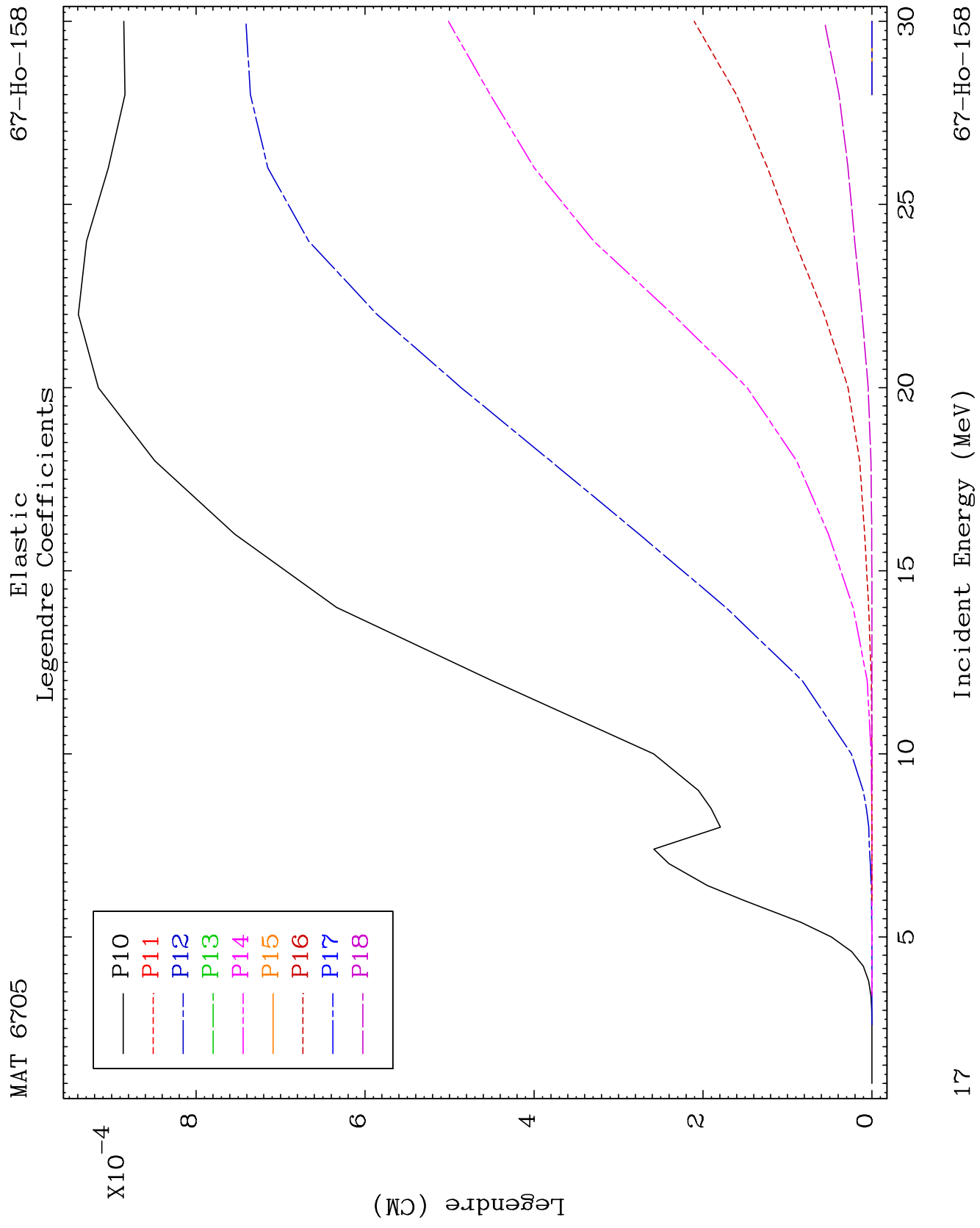
(n,  $\alpha$ ) Levels  
293 Kelvin Cross Sections

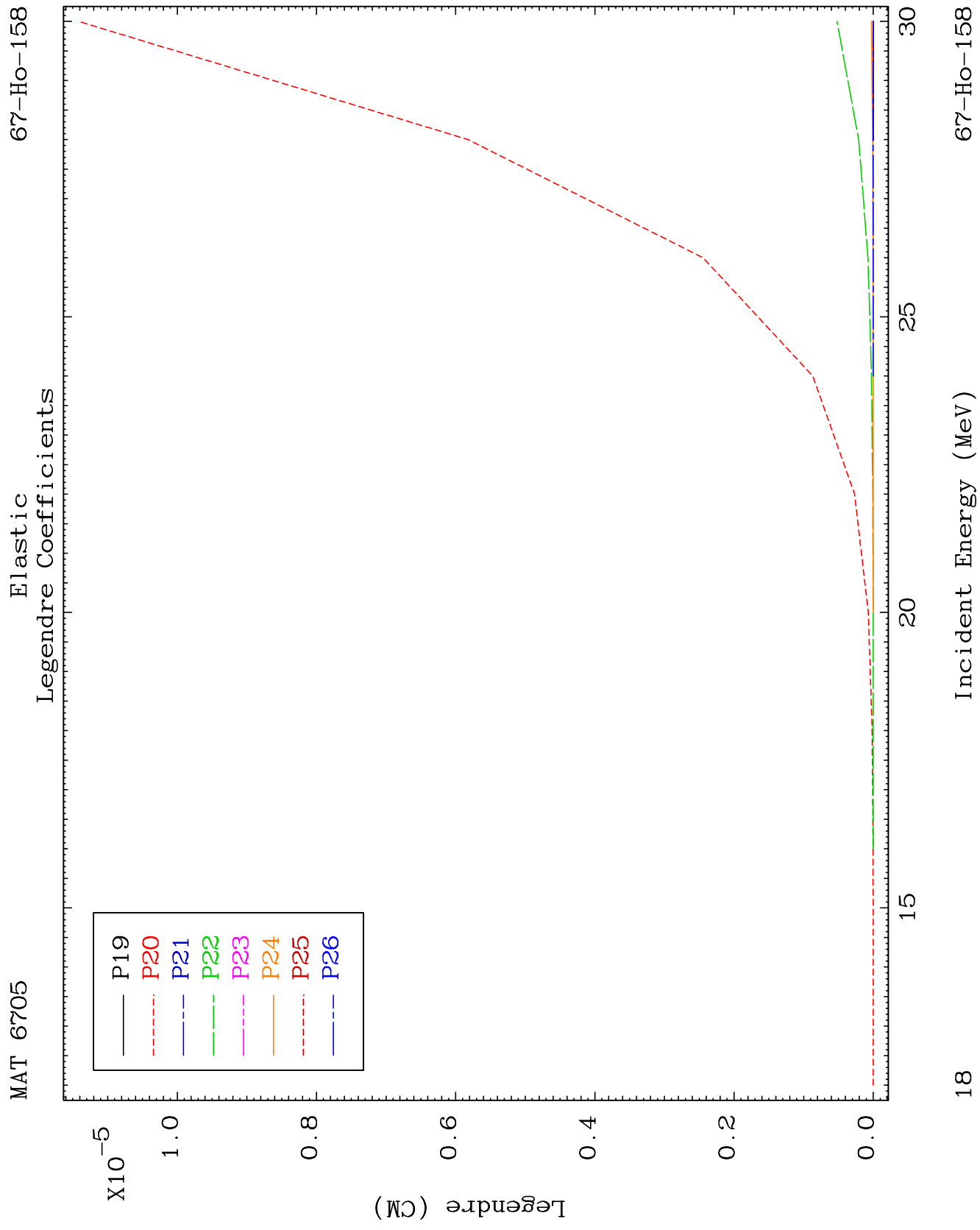












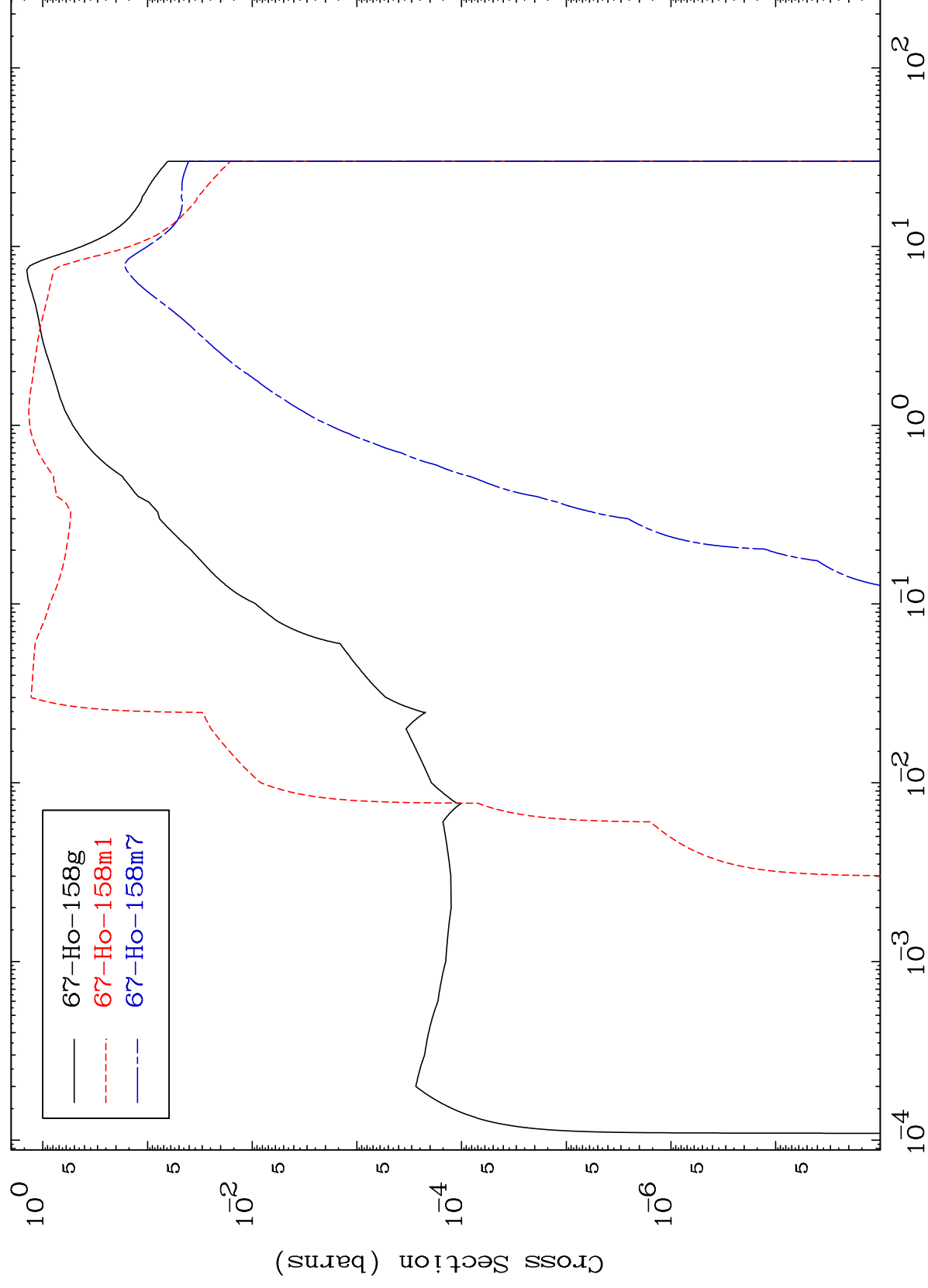
MAT 6705

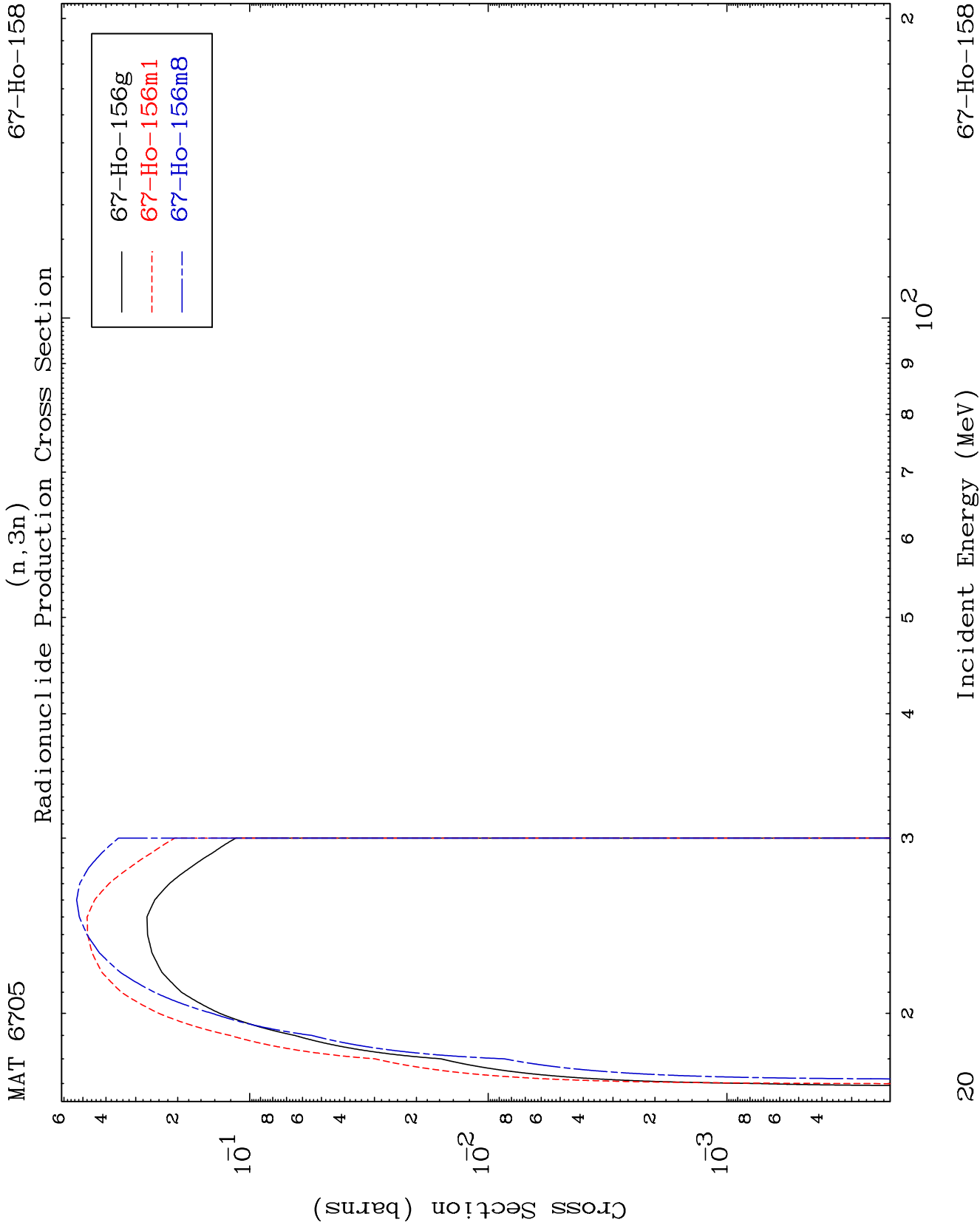
67-Ho-158

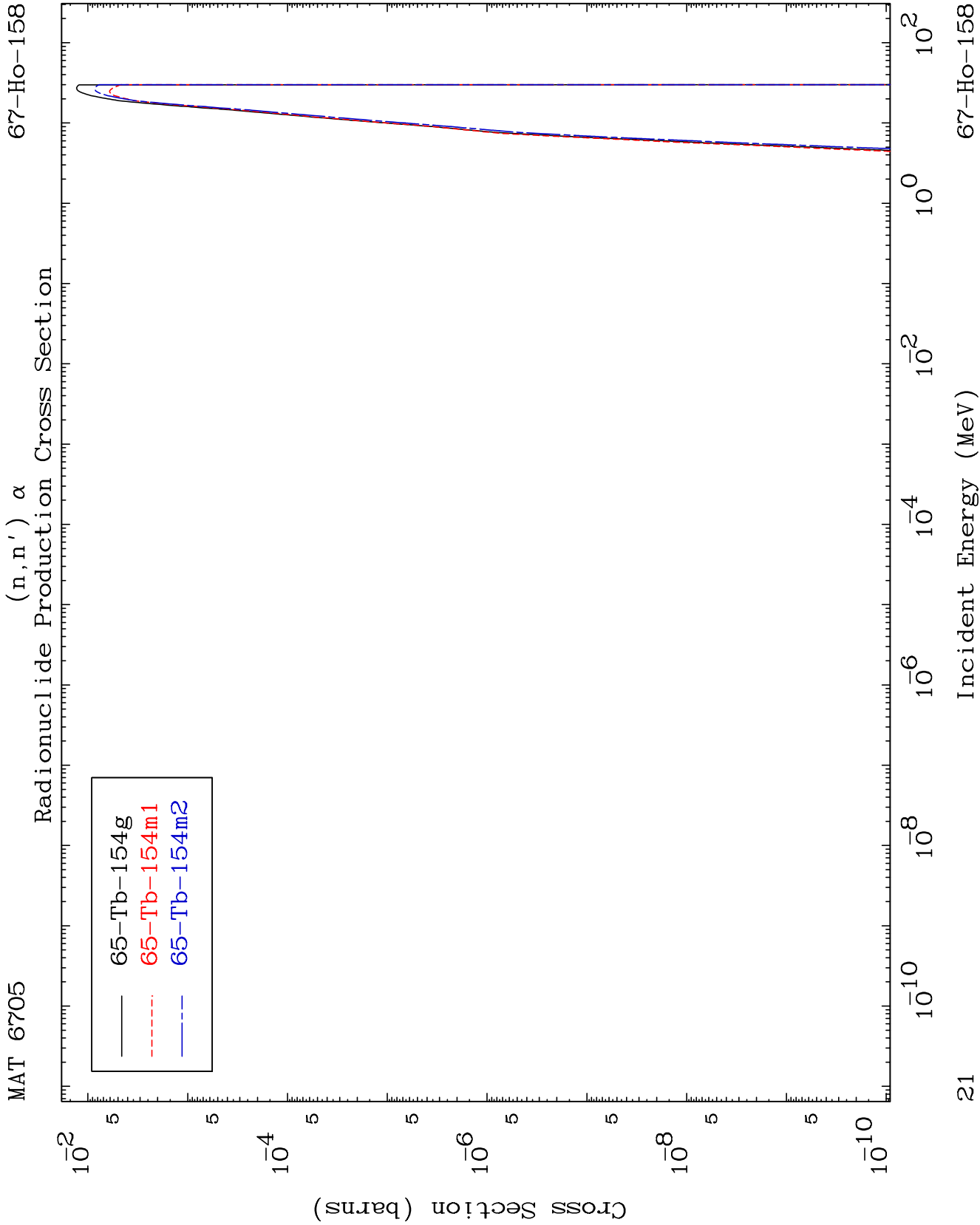
# Inelastic Radionuclide Production Cross Section

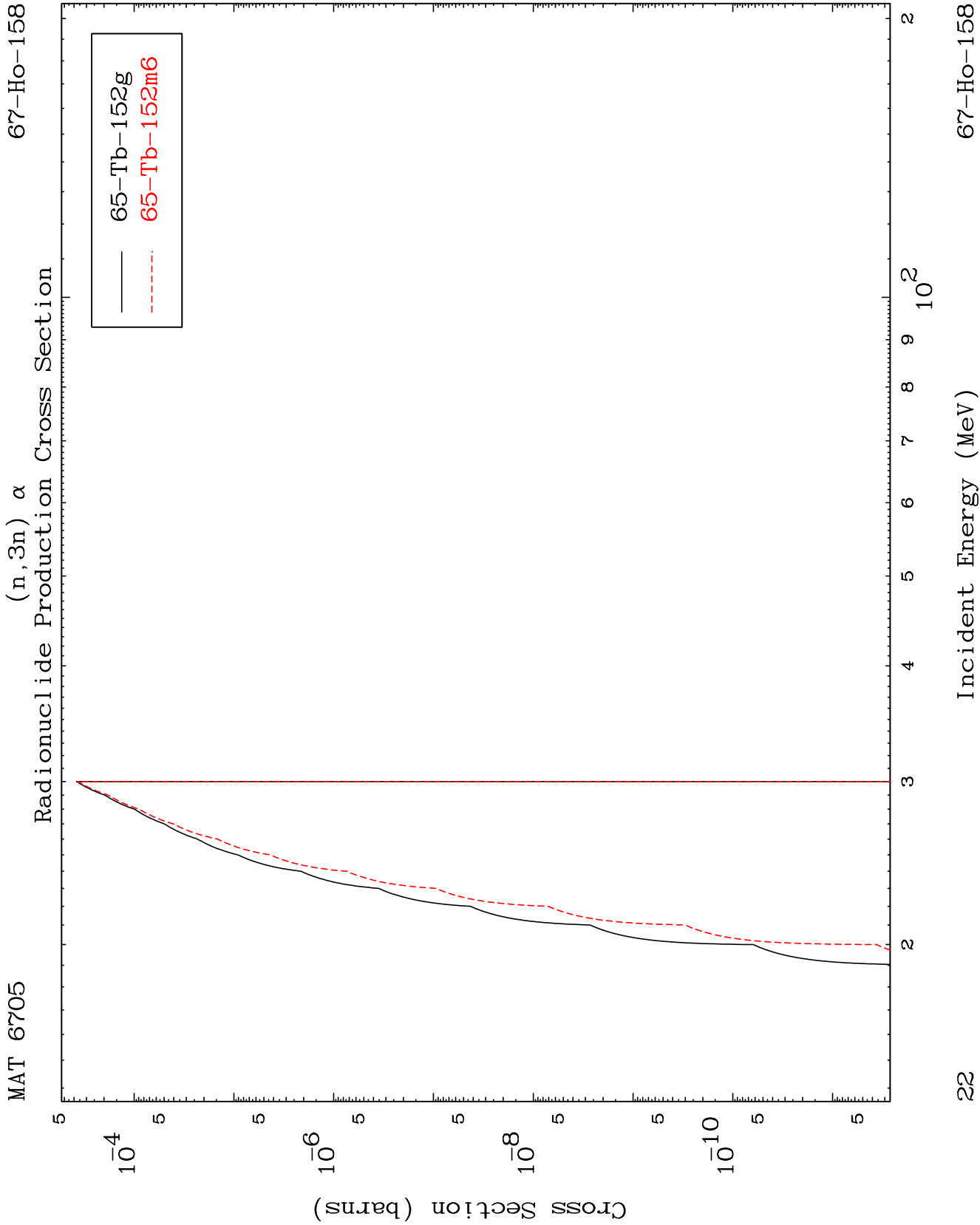
67-Ho-158

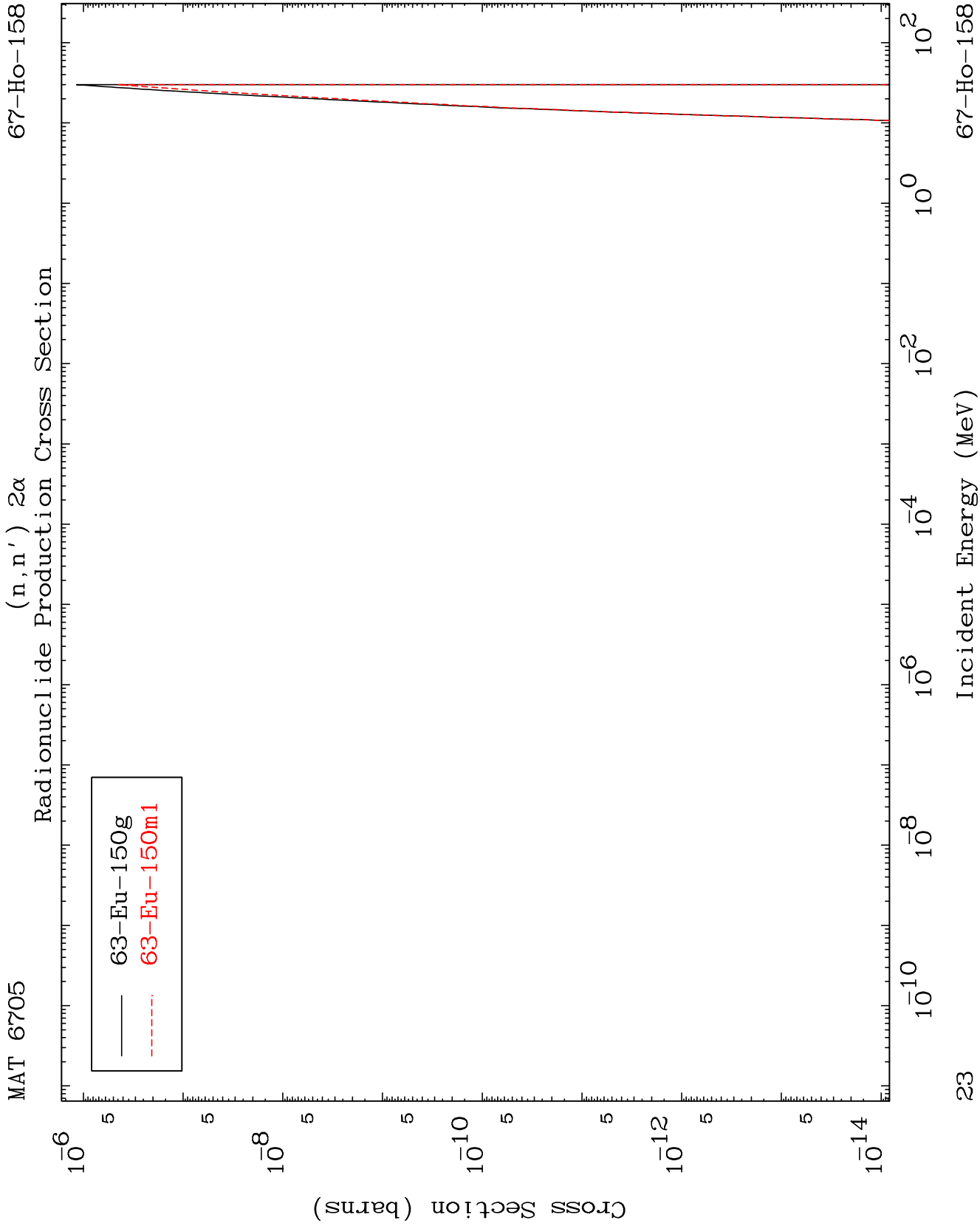
Incident Energy (MeV)





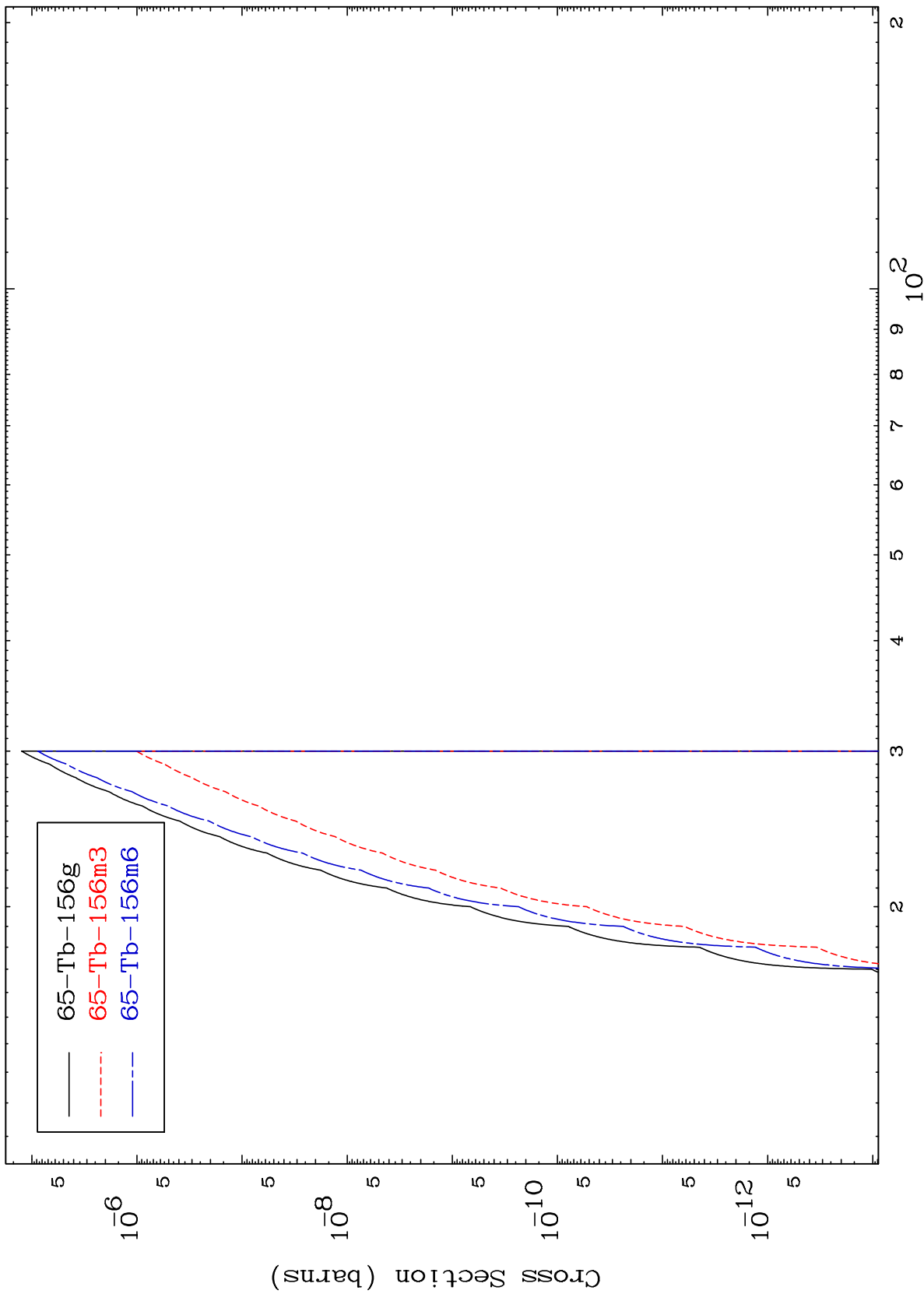


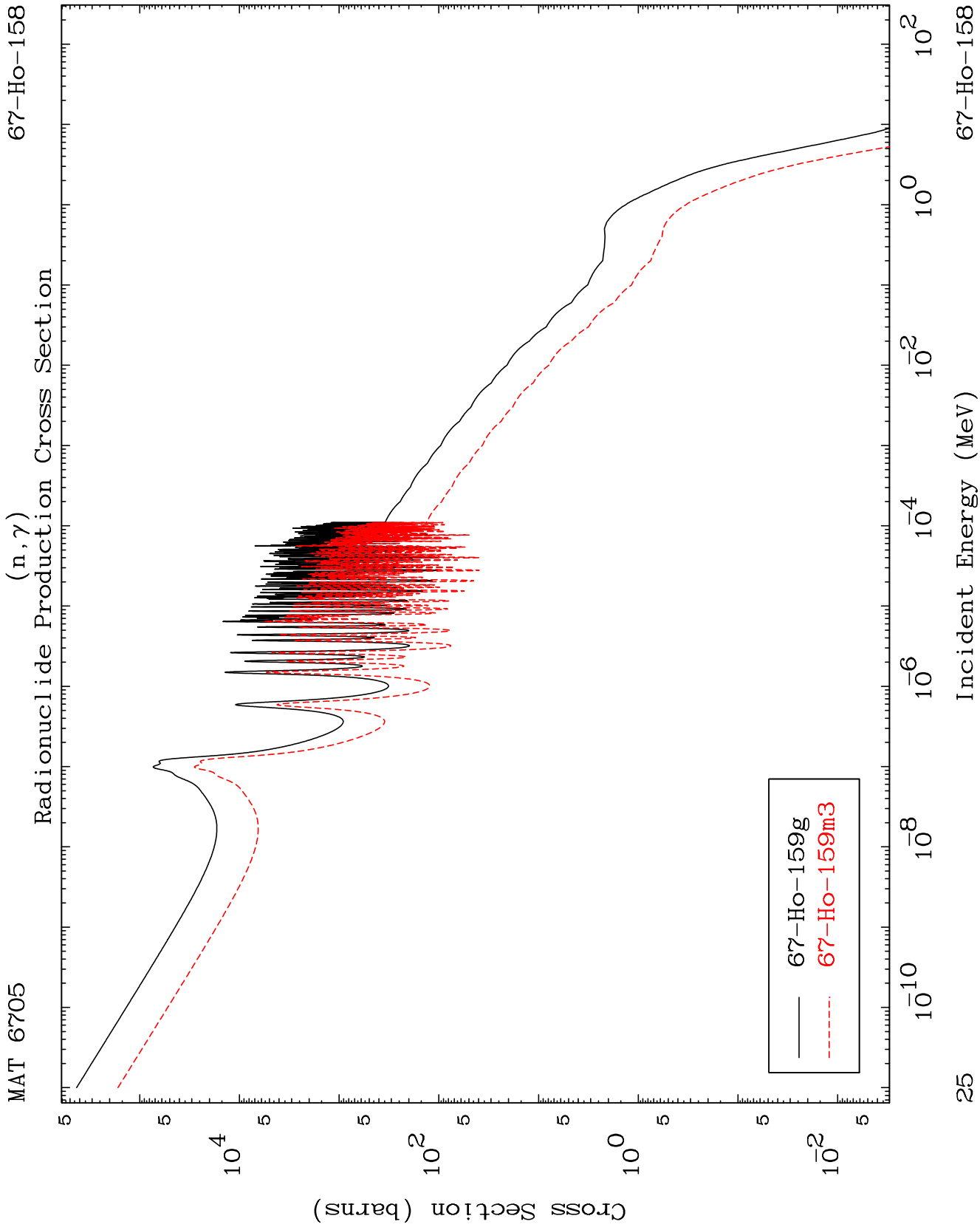






(n,2n) p  
Radionuclide Production Cross Section

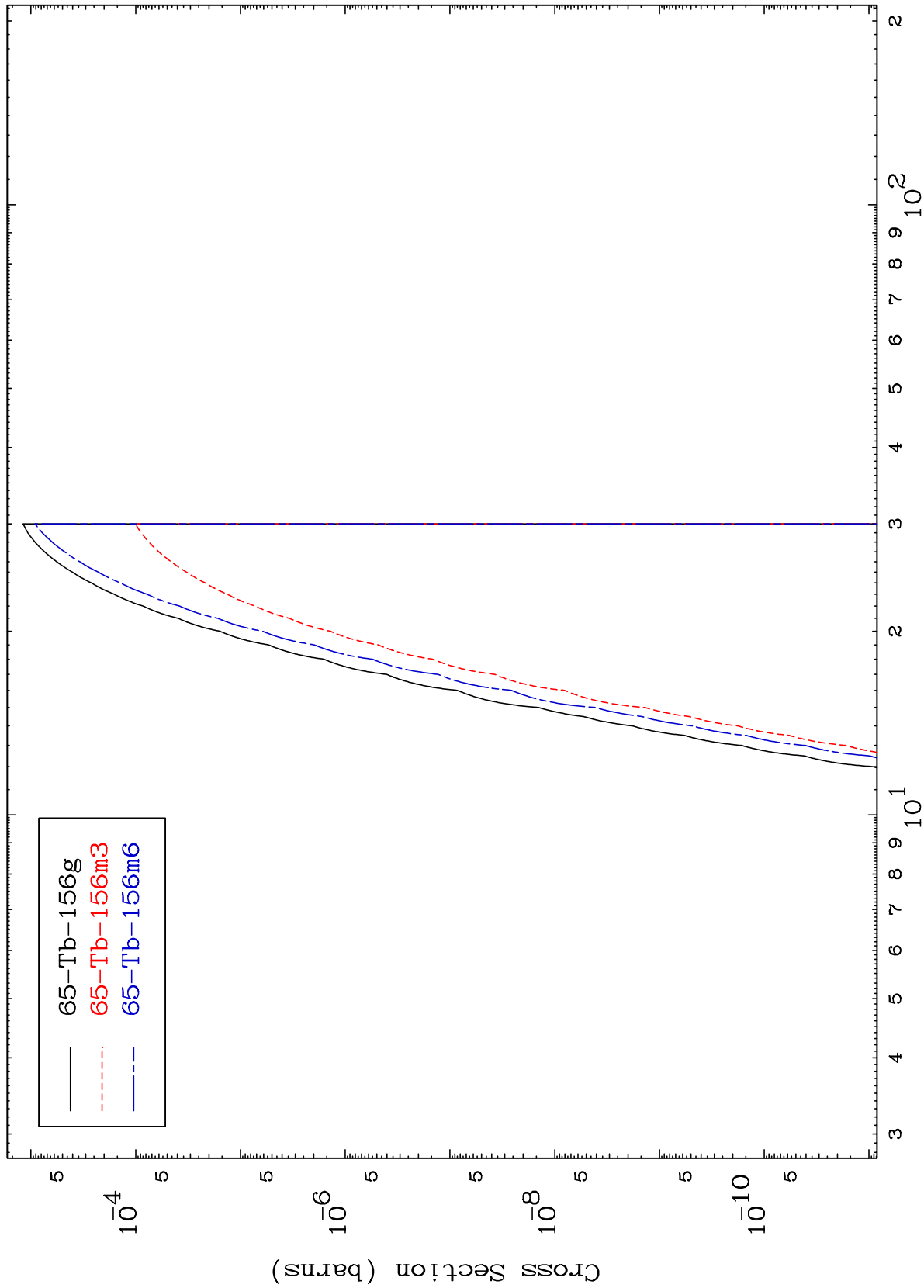




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Radionuclide Production Cross Section  
(n,He-3)



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Incident Energy (MeV)

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