

Program EVALPLOT  
(Version 2015-2)

by

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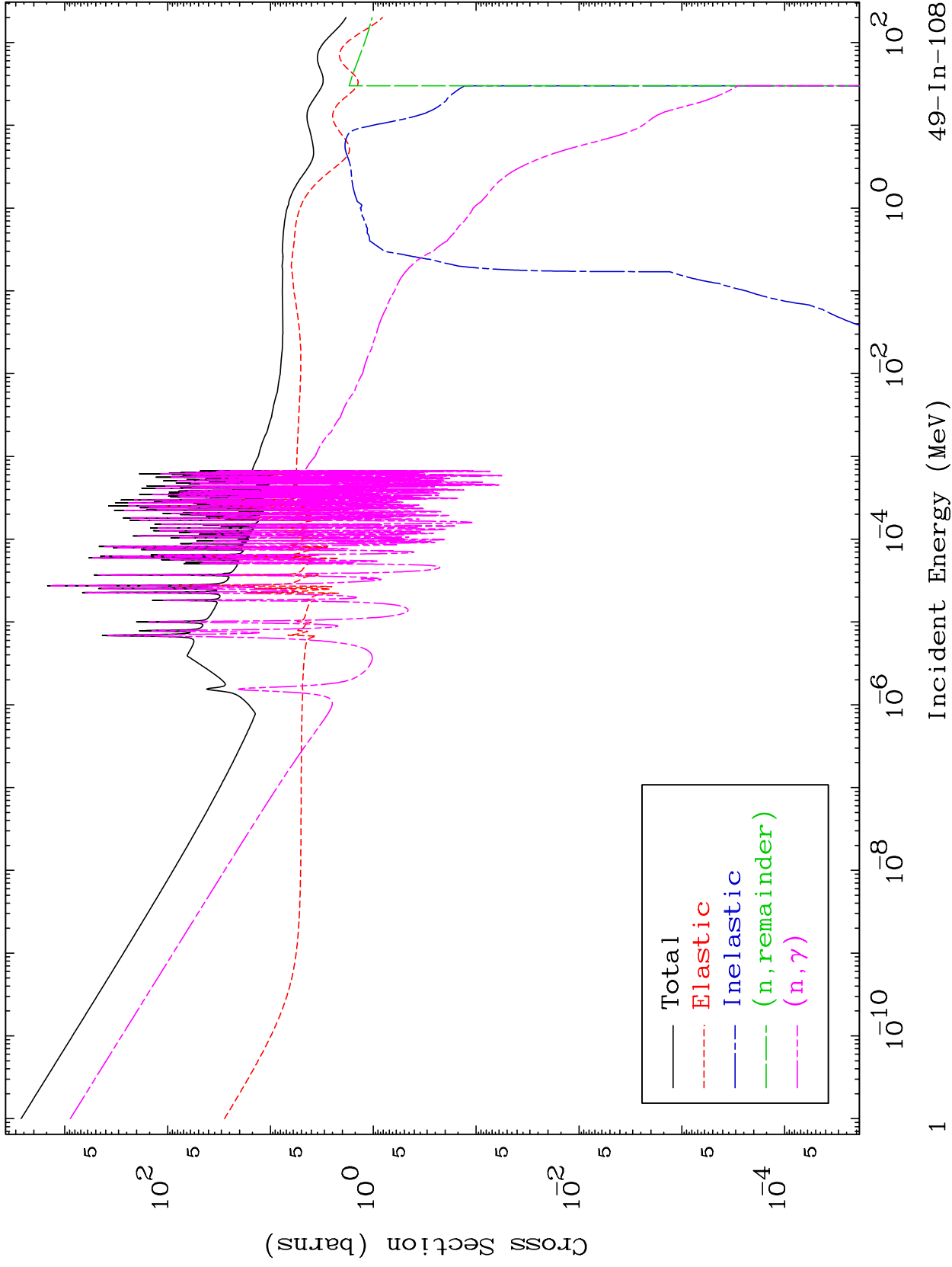
Press Mouse Button to Start

MAT 4911

Major

293 Kelvin Cross Sections

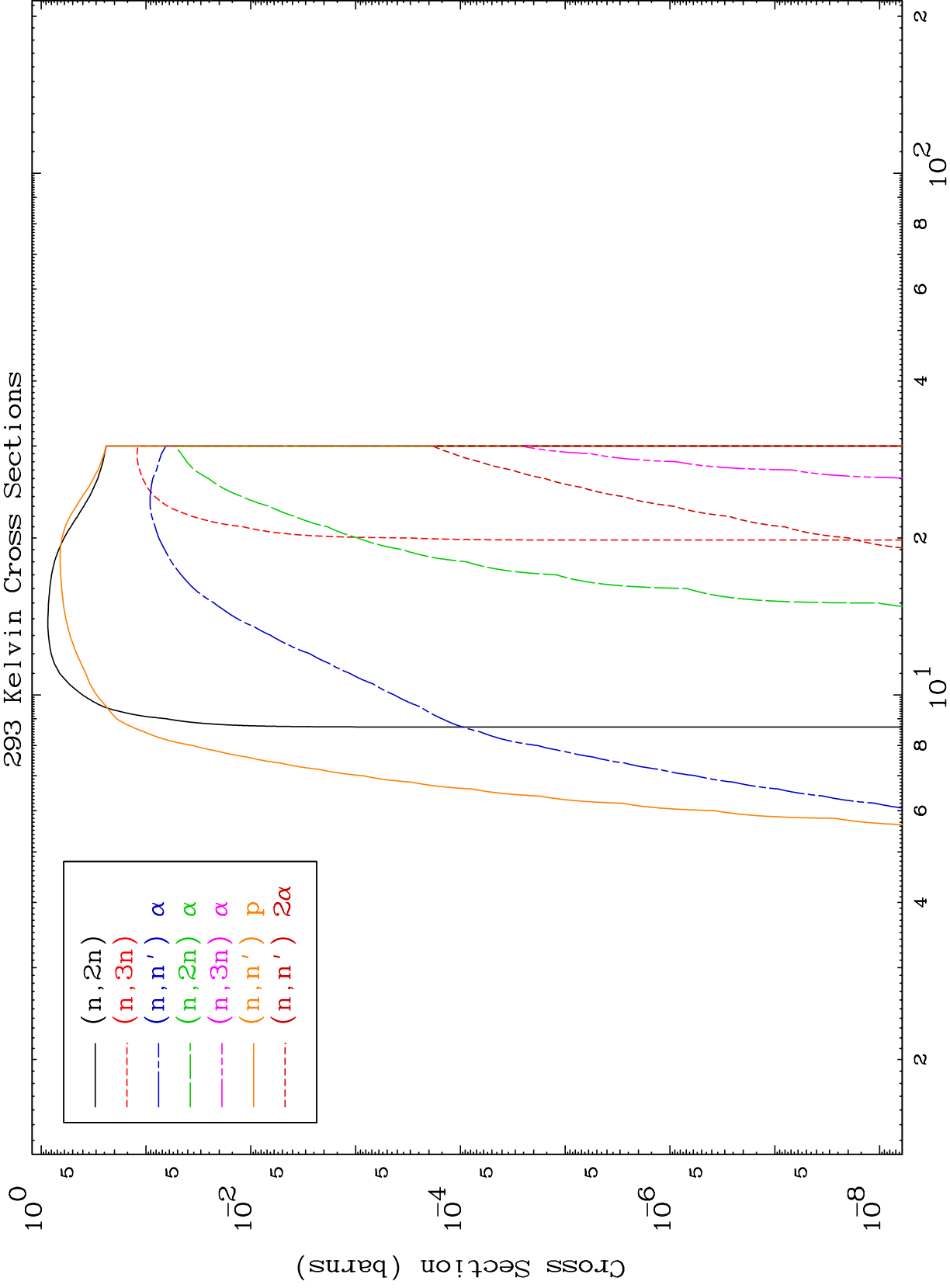
49-In-108



MAT 4911

Neutron Production  
293 Kelvin Cross Sections

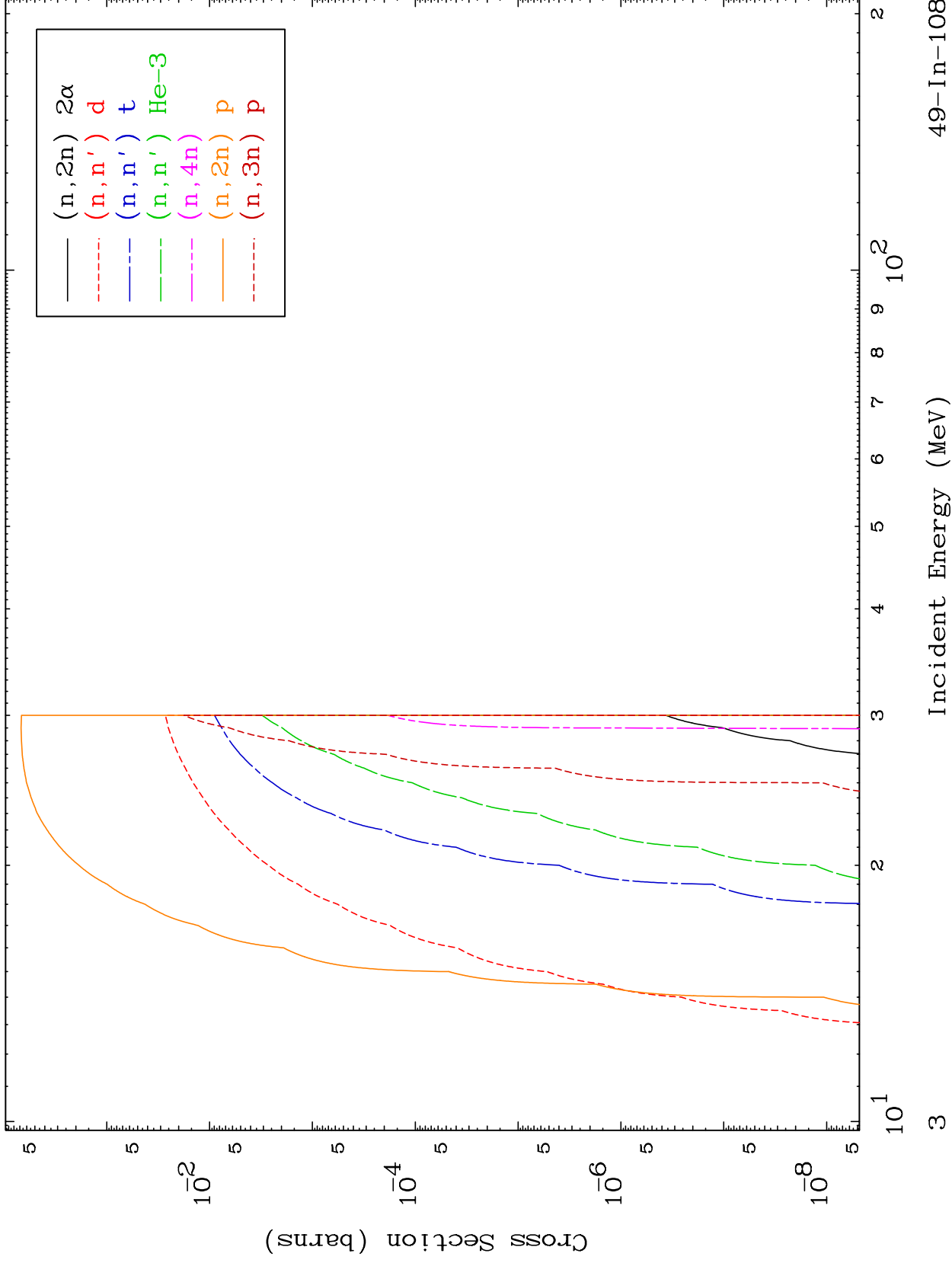
49-In-108



MAT 4911

Neutron Production  
293 Kelvin Cross Sections

49-In-108

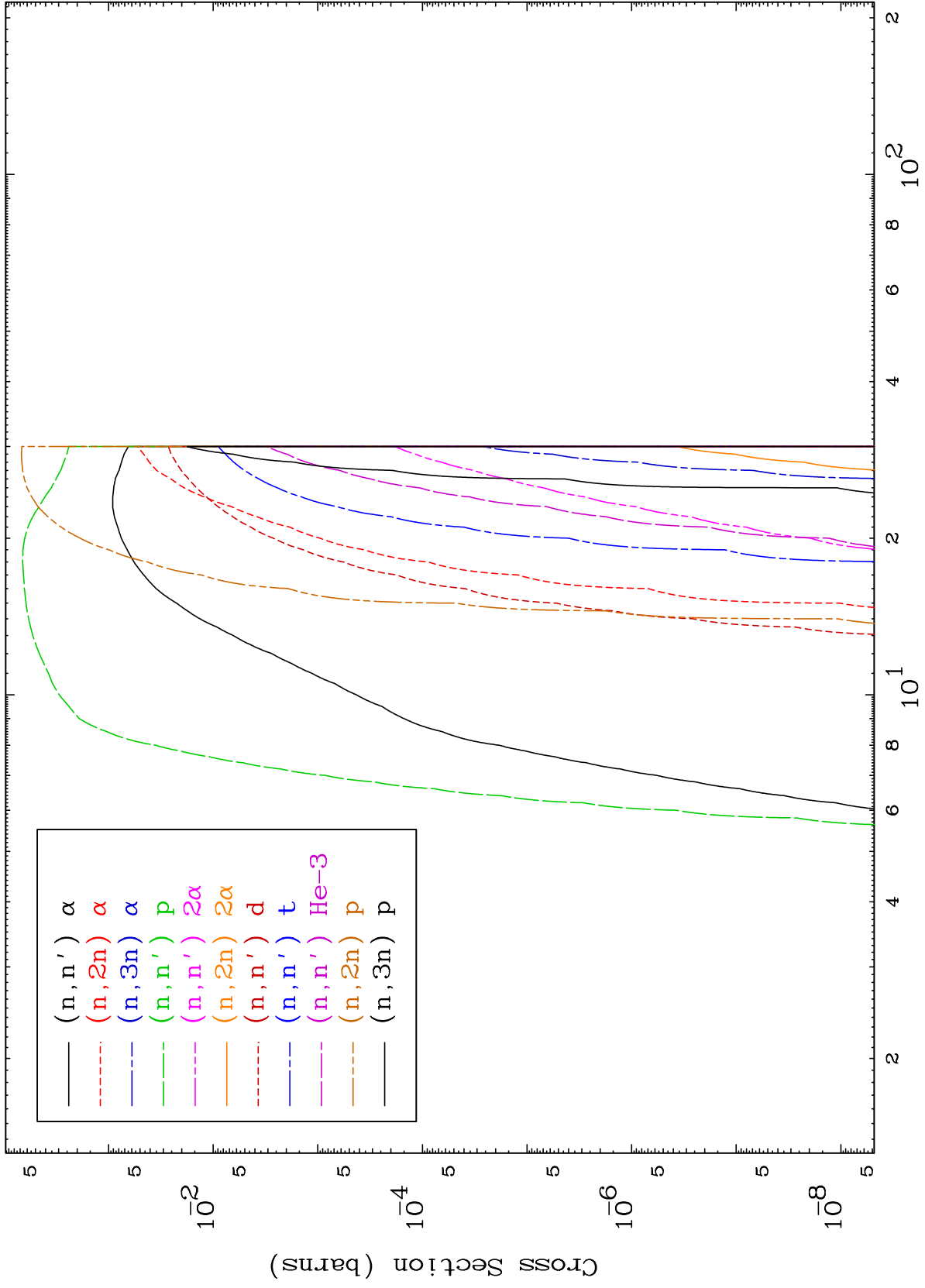


49-In-108

MAT 4911

Charged Particle  
293 Kelvin Cross Sections

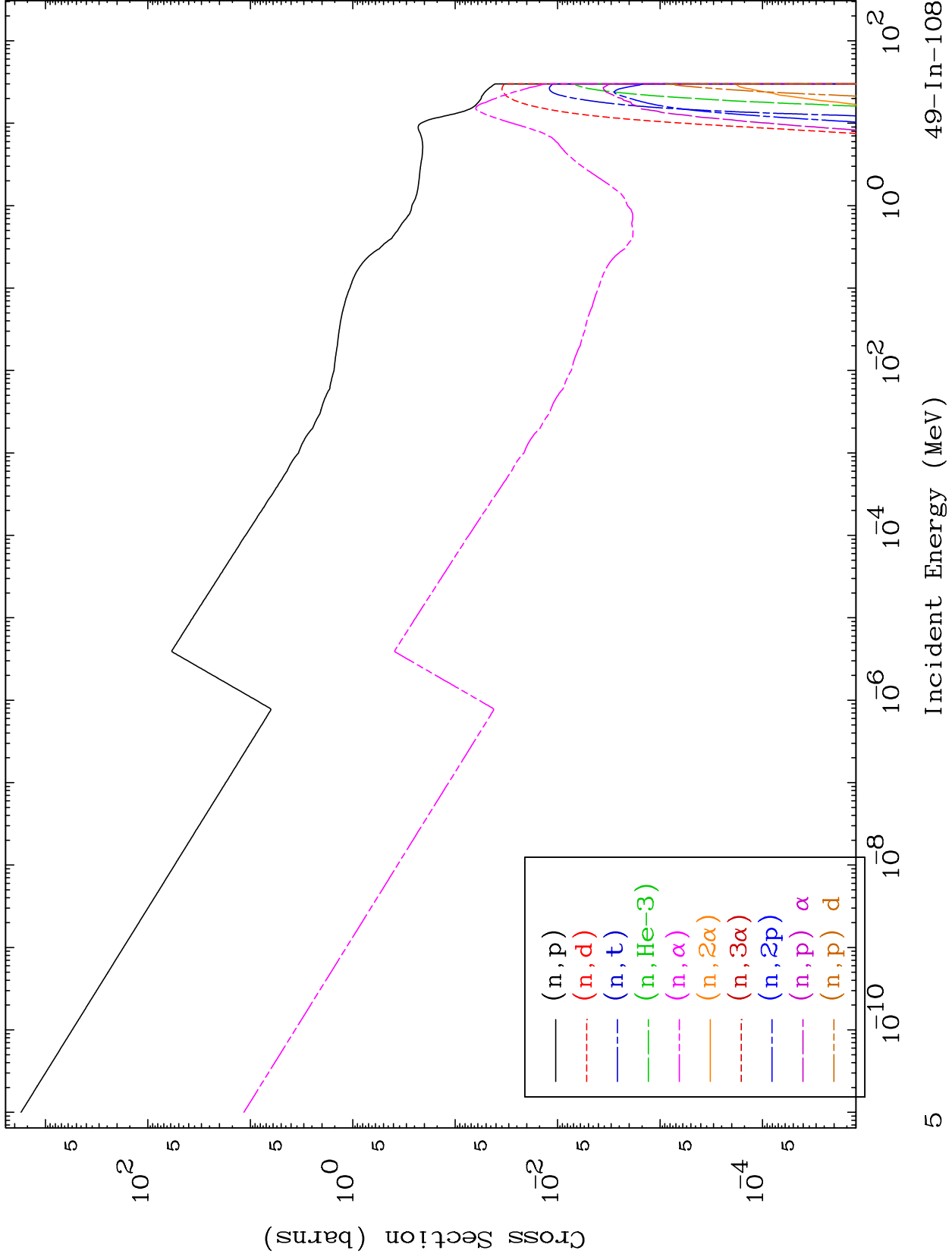
49-In-108



MAT 4911

Charged Particle  
293 Kelvin Cross Sections

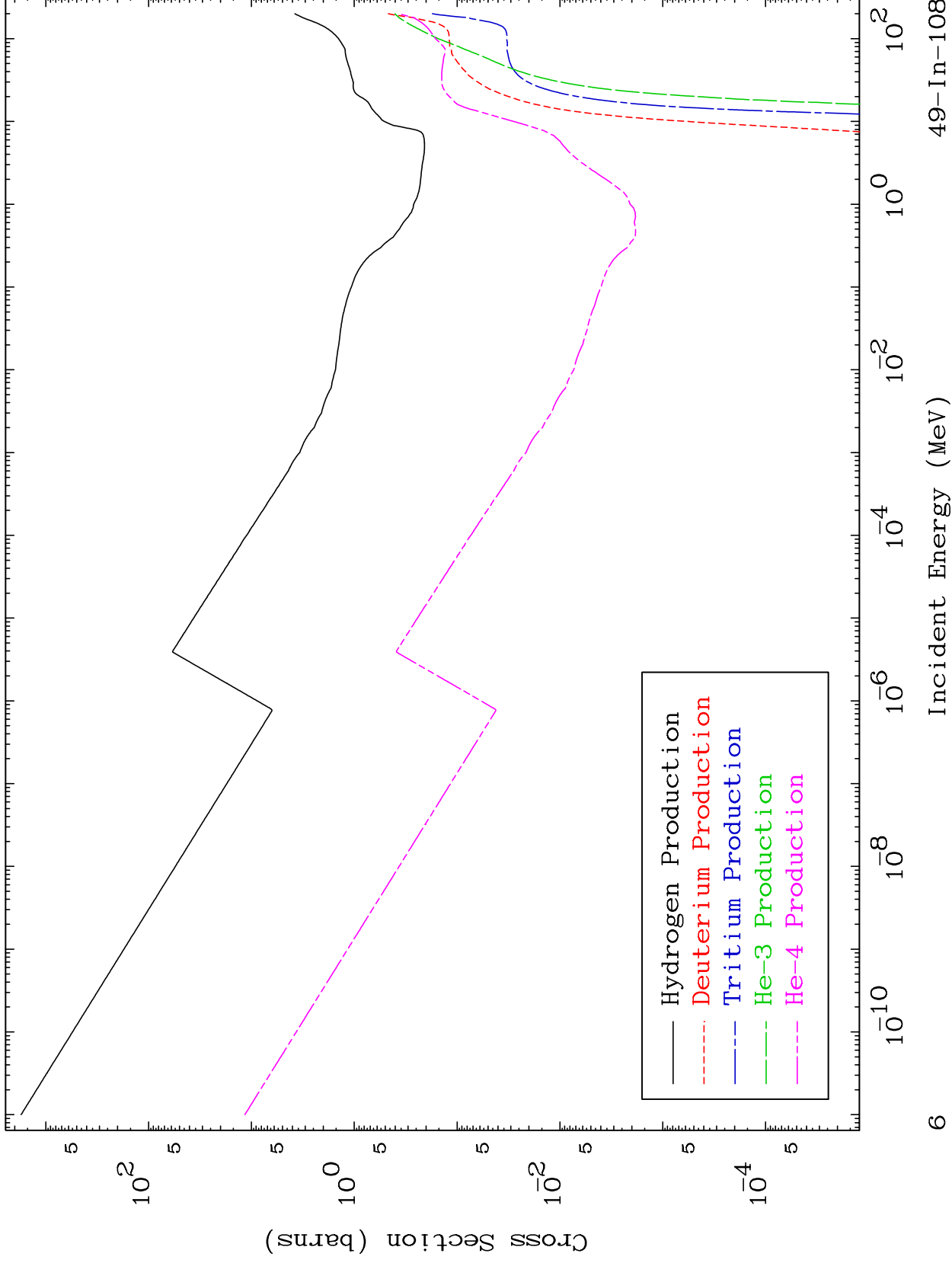
49-In-108



MAT 4911

Particle Production  
293 Kelvin Cross Sections

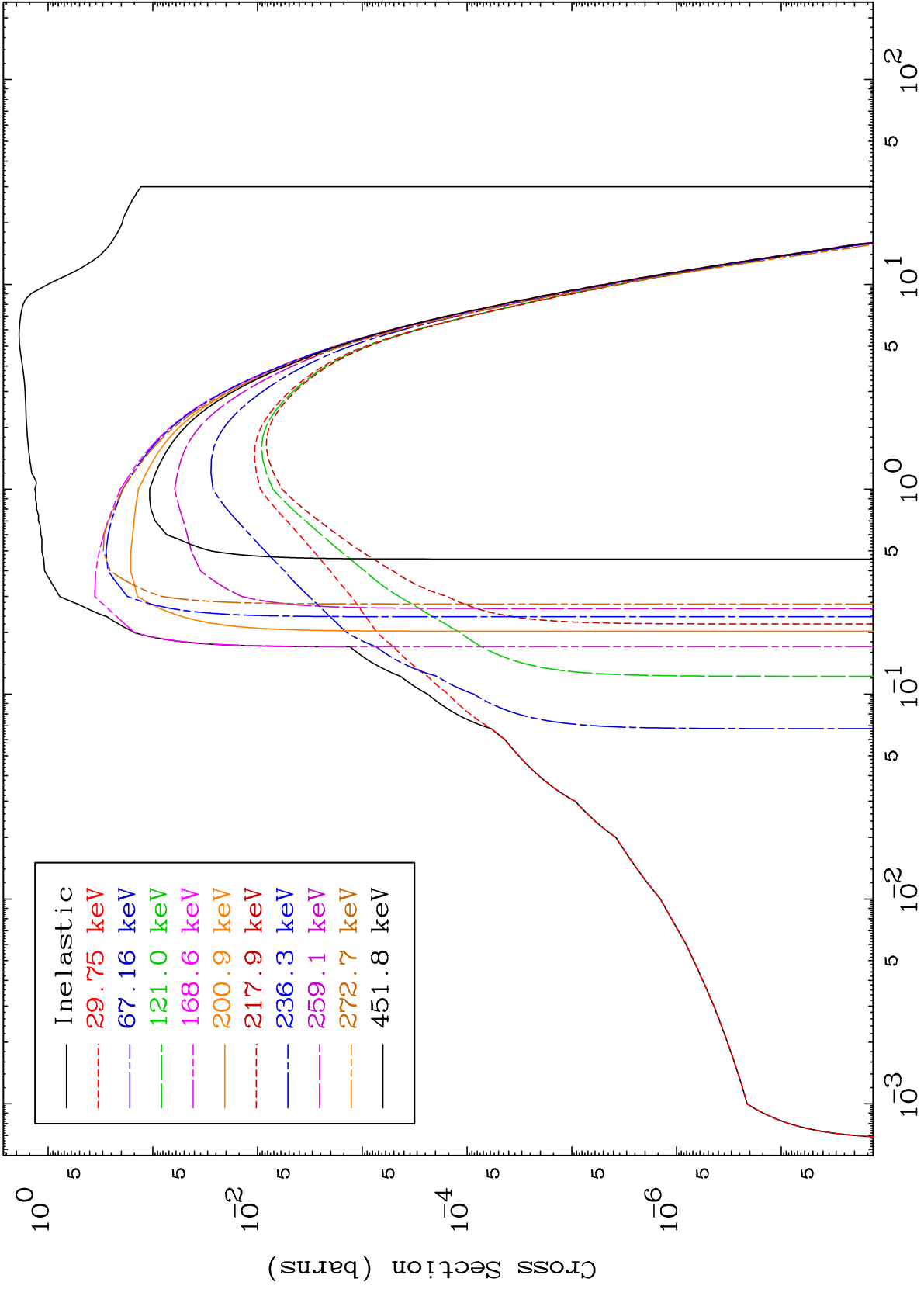
49-In-108



MAT 4911

(n,n') Level  
293 Kelvin Cross Sections

49-In-108

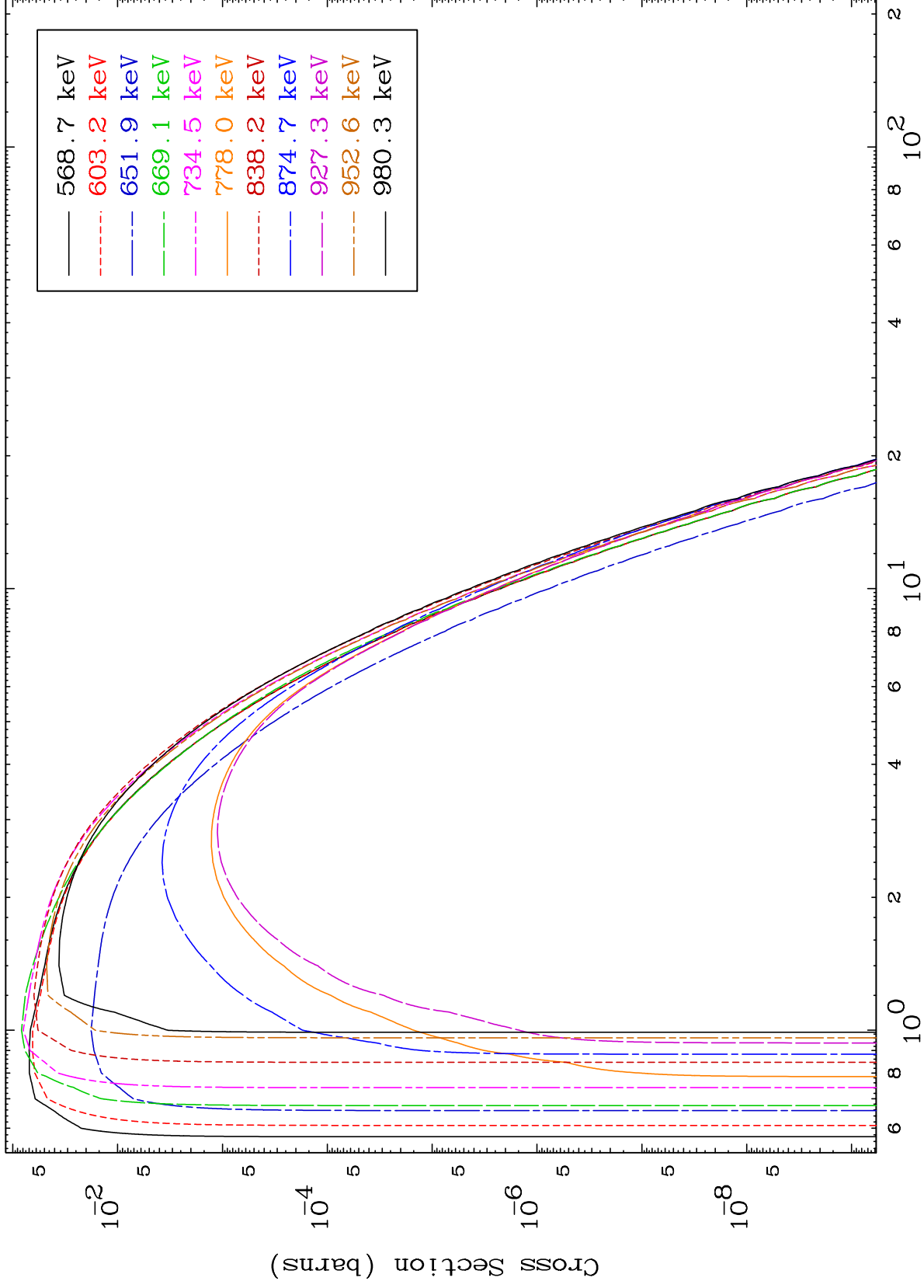




MAT 4911

(n,n') Level  
293 Kelvin Cross Sections

49-In-108



8

Incident Energy (MeV)

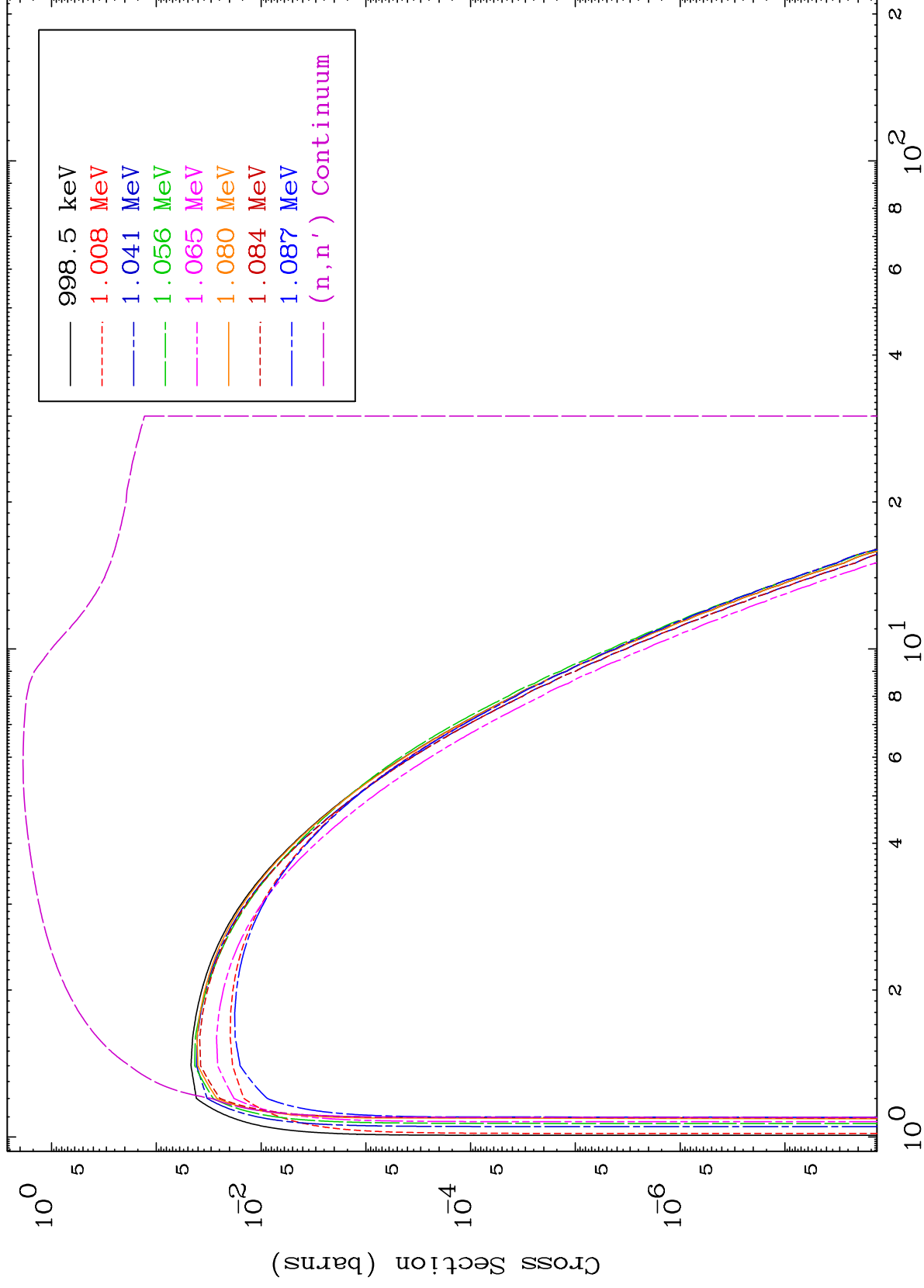
49-In-108

MAT 4911

(n,n') Level

293 Kelvin Cross Sections

49-In-108



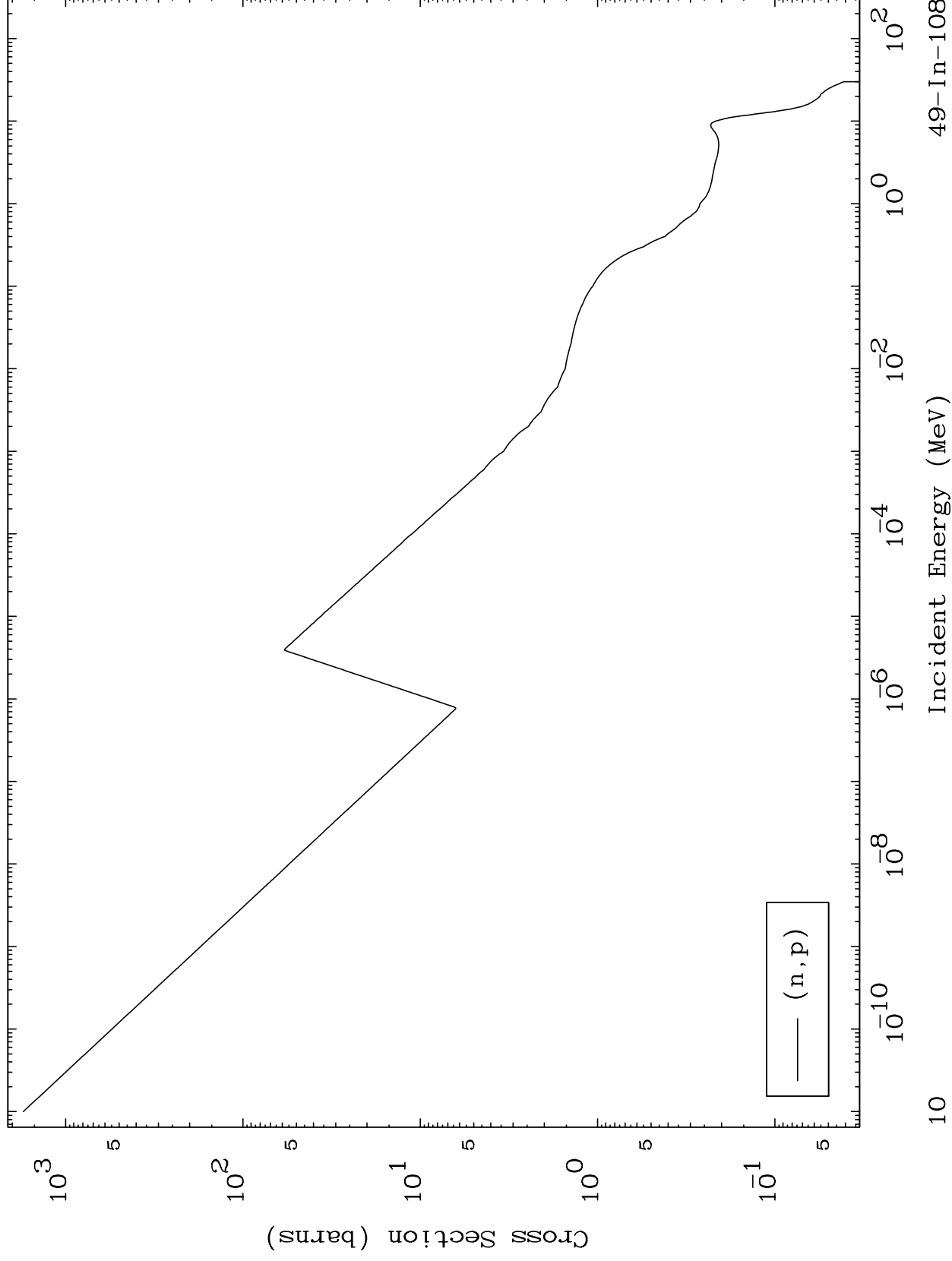
Incident Energy (MeV)

49-In-108

MAT 4911

(n,p) Levels  
293 Kelvin Cross Sections

49-In-108



10

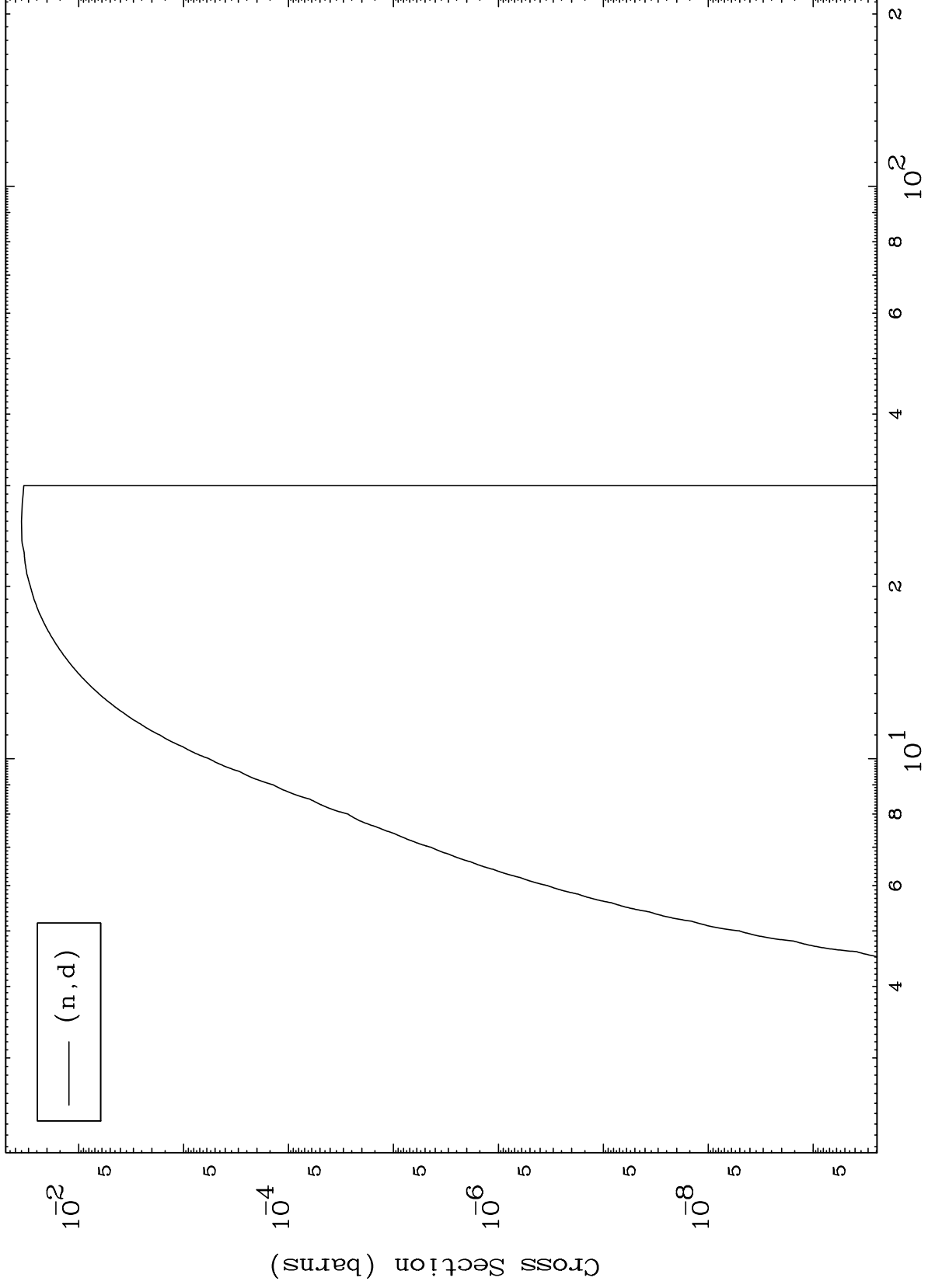
Incident Energy (MeV)

49-In-108

MAT 4911

(n,d) Levels  
293 Kelvin Cross Sections

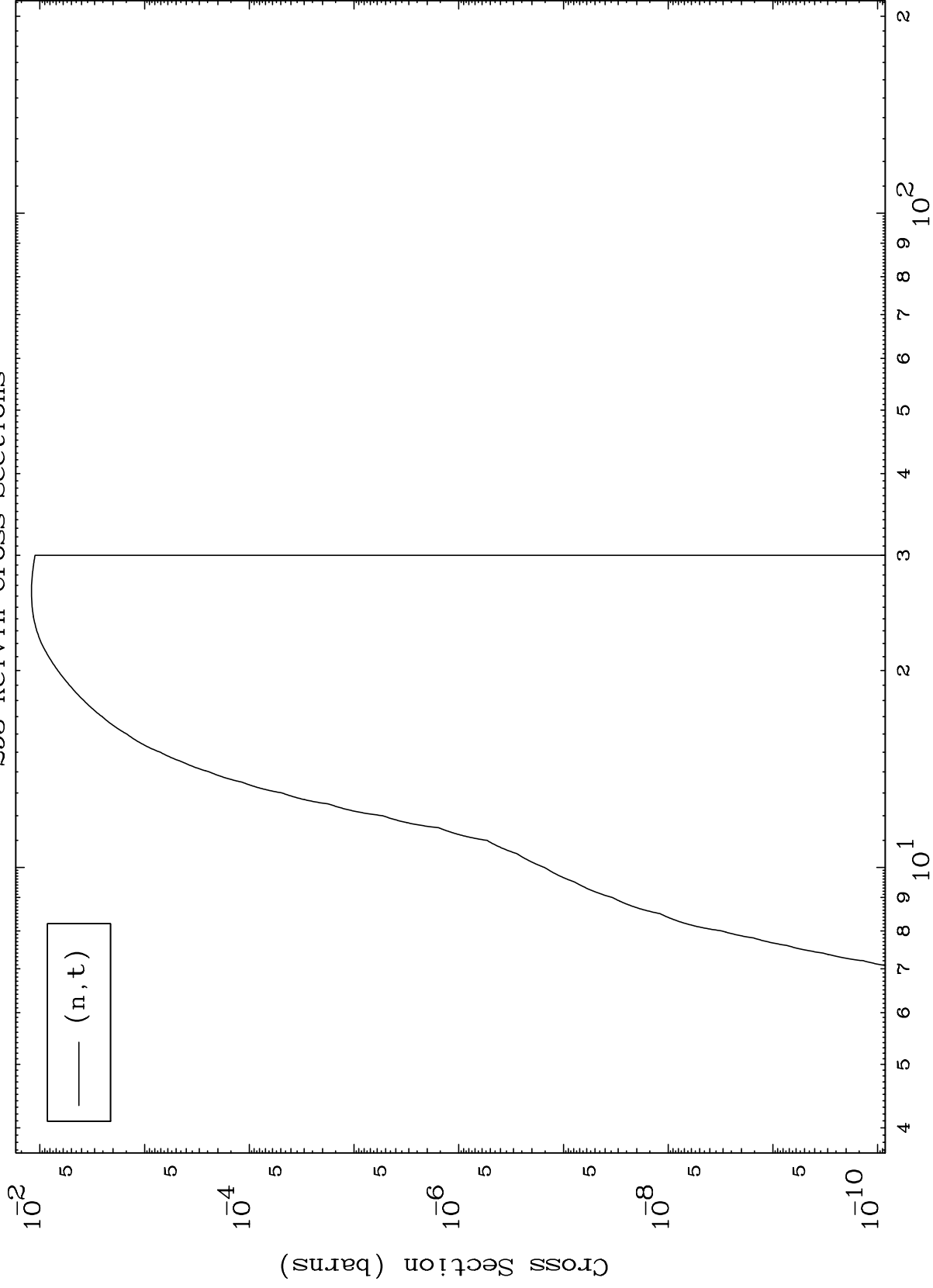
49-In-108



MAT 4911

(n,t) Levels  
293 Kelvin Cross Sections

49-In-108



12

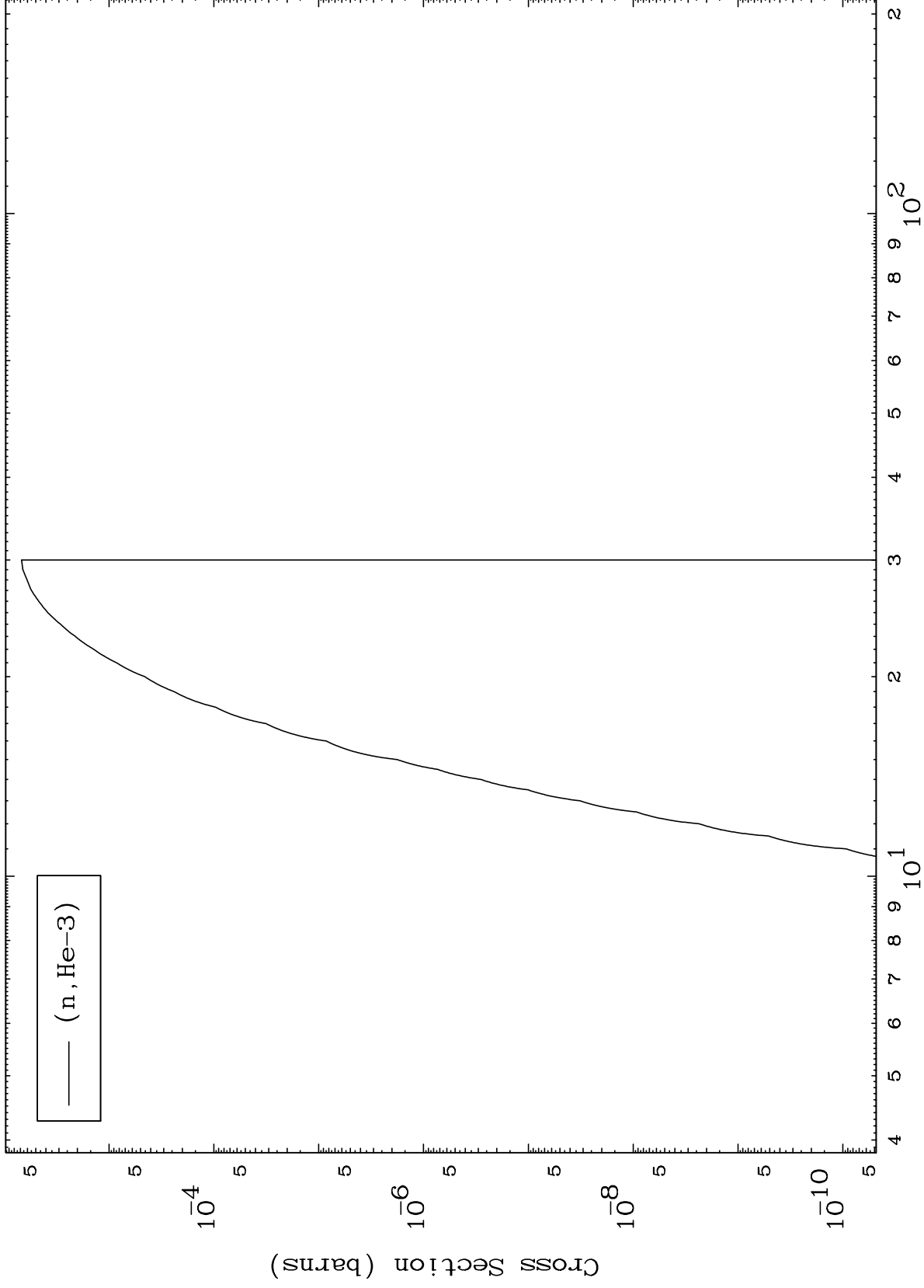
Incident Energy (MeV)

49-In-108

MAT 4911

(n,He3) Levels  
293 Kelvin Cross Sections

49-In-108



13

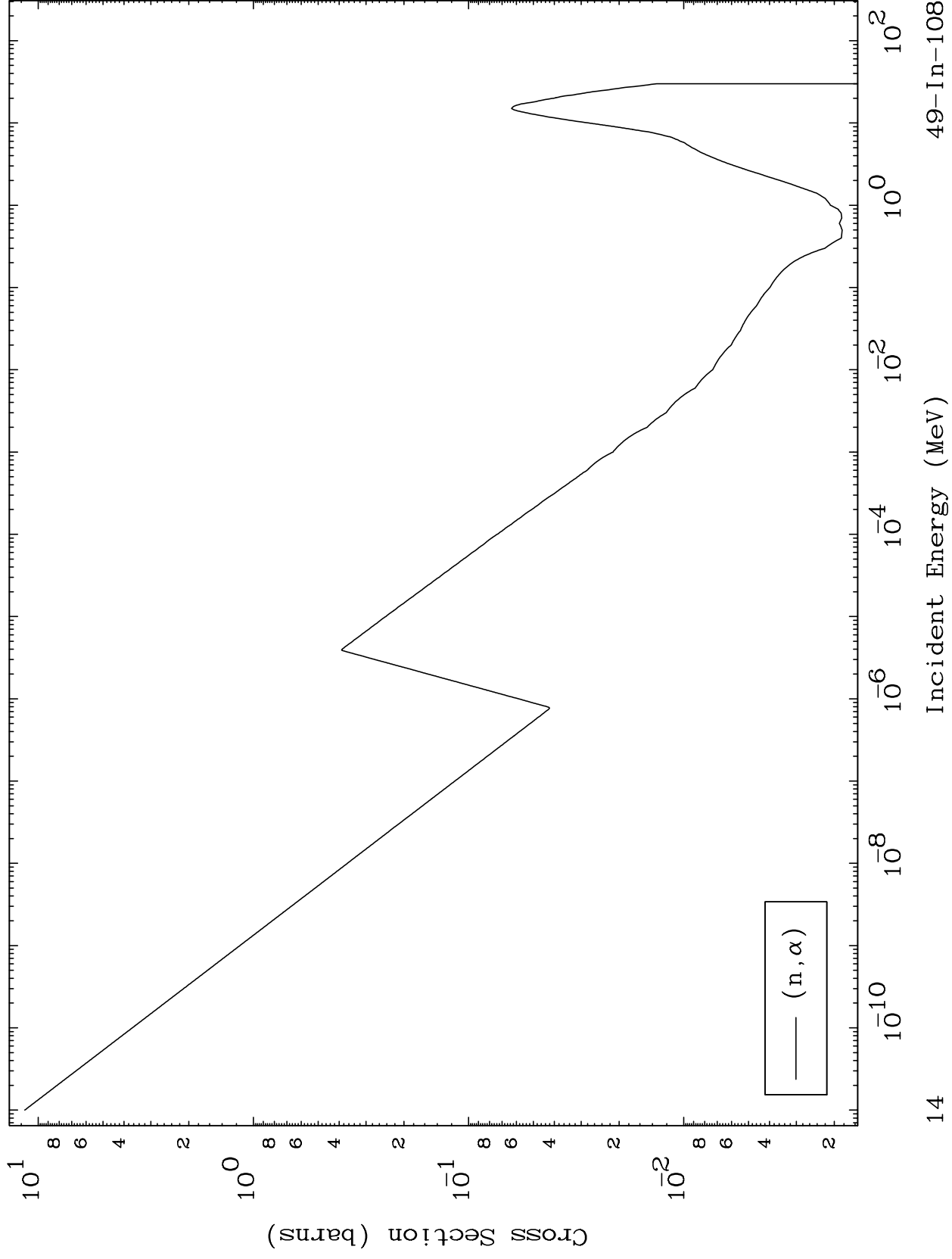
Incident Energy (MeV)

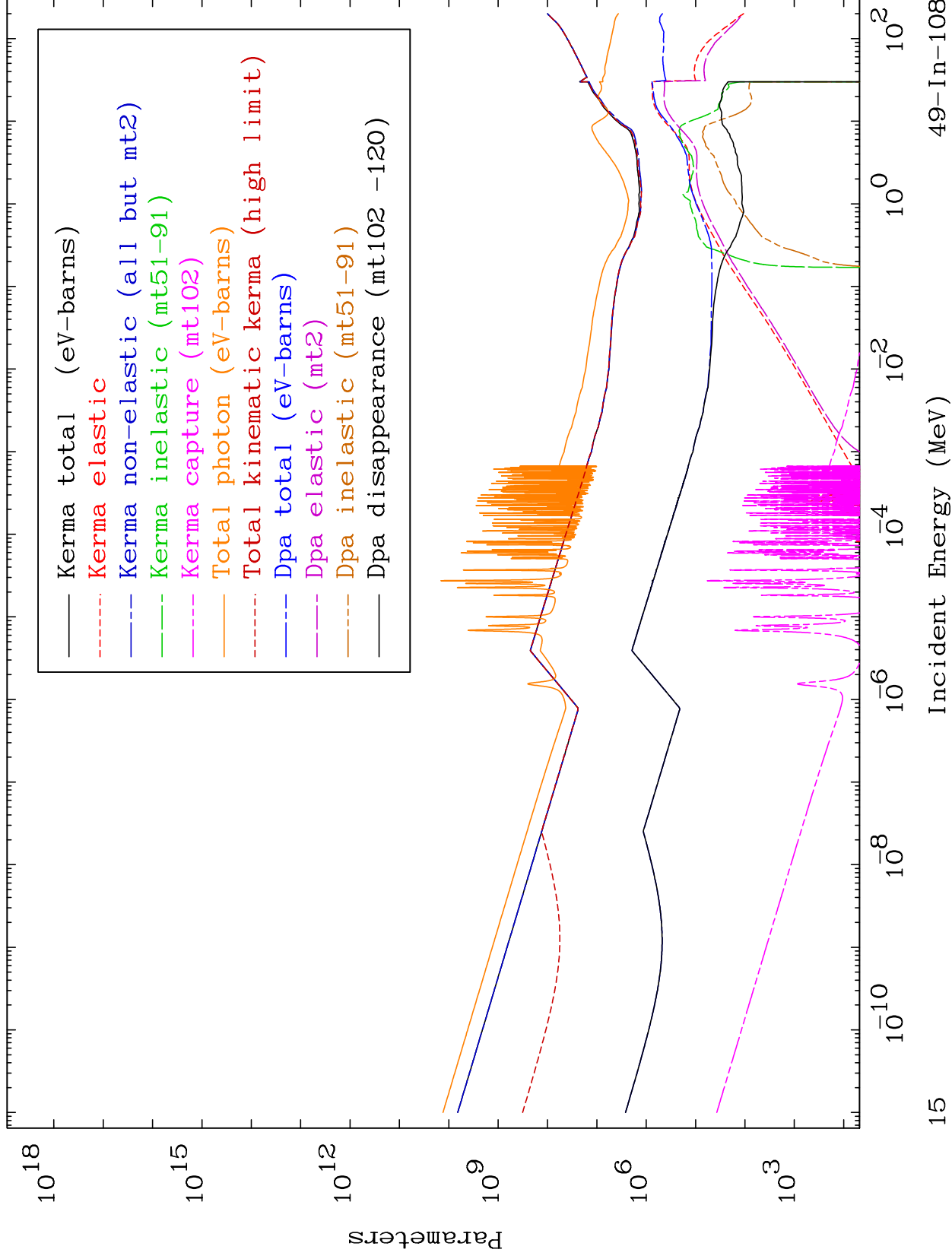
49-In-108

MAT 4911

(n,  $\alpha$ ) Levels  
293 Kelvin Cross Sections

49-In-108





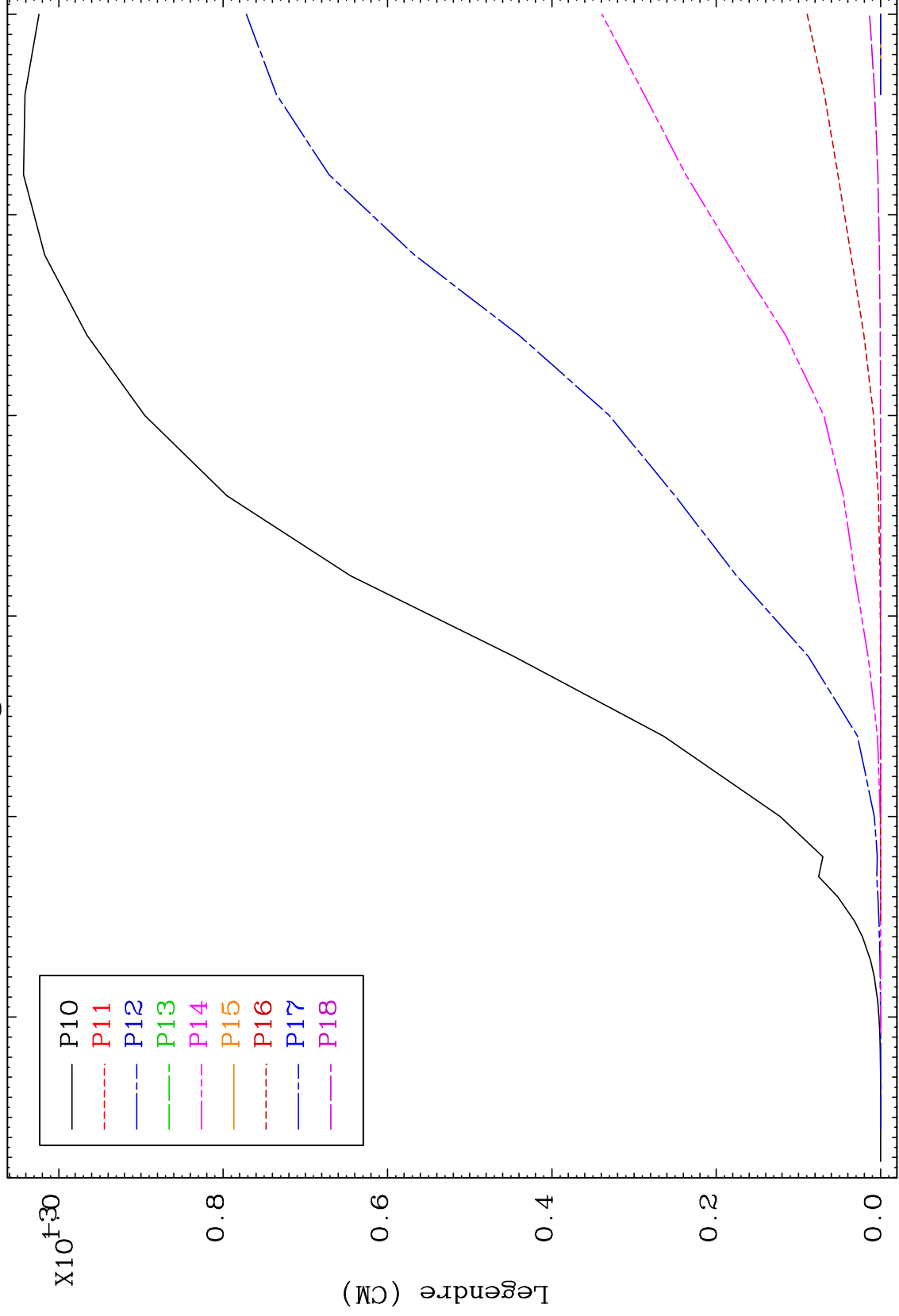




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Elastic Legendre Coefficients

49-In-108



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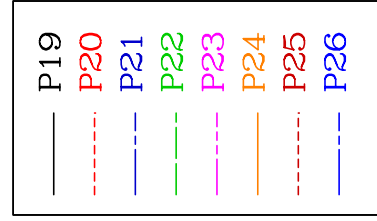
Incident Energy (MeV)

49-In-108

MAT 4911

Elastic  
Legendre Coefficients

49-In-108



$\times 10^{-7}$

Legendre (CM)

6

4

2

0

15

20

25

30

18

Incident Energy (MeV)

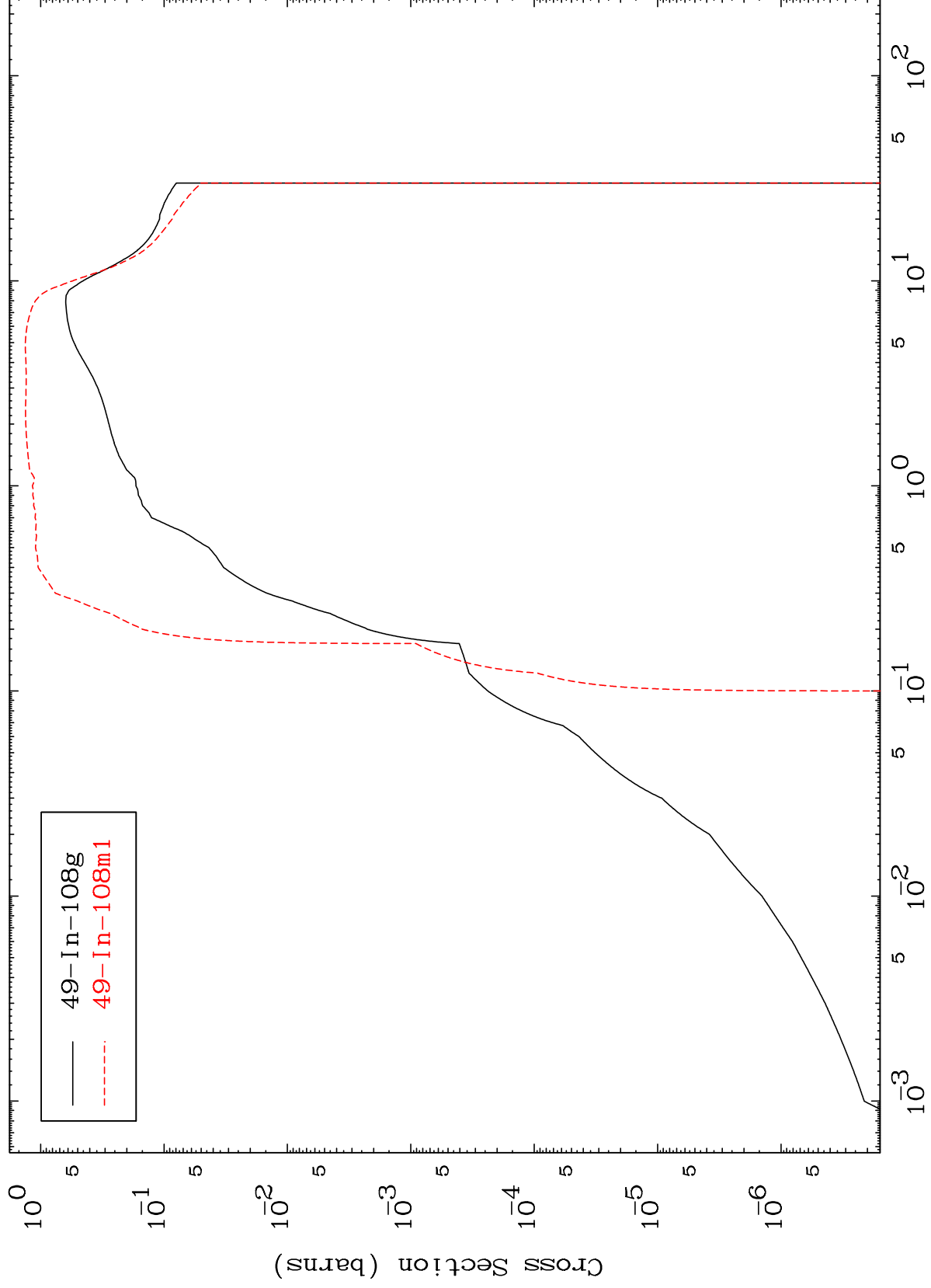
49-In-108



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49-In-108

Inelastic  
Radionuclide Production Cross Section



— 49-In-108g  
- - - 49-In-108m1

19

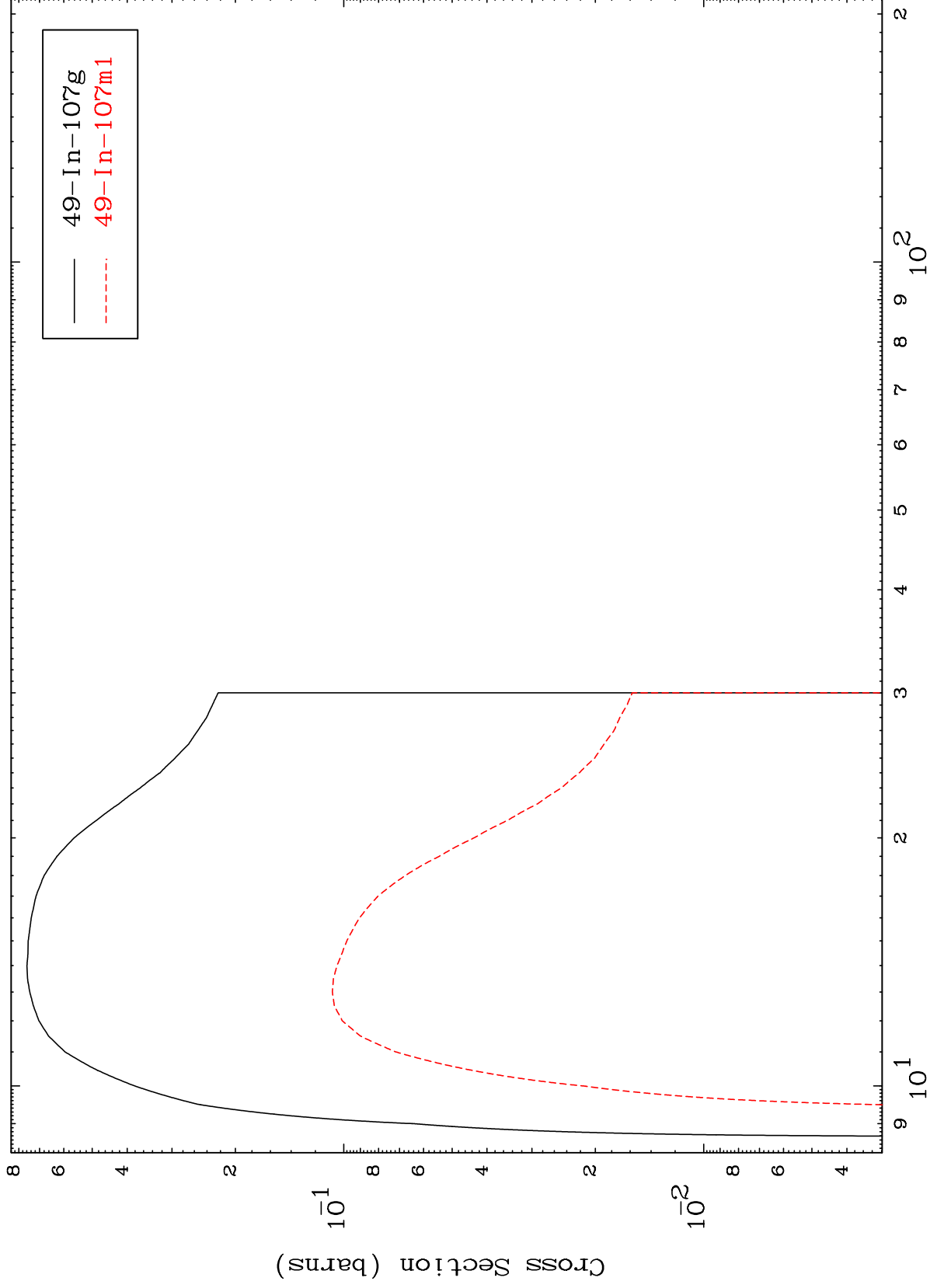
49-In-108

Incident Energy (MeV)

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49-In-108

(n,2n)  
Radionuclide Production Cross Section



49-In-108

Incident Energy (MeV)

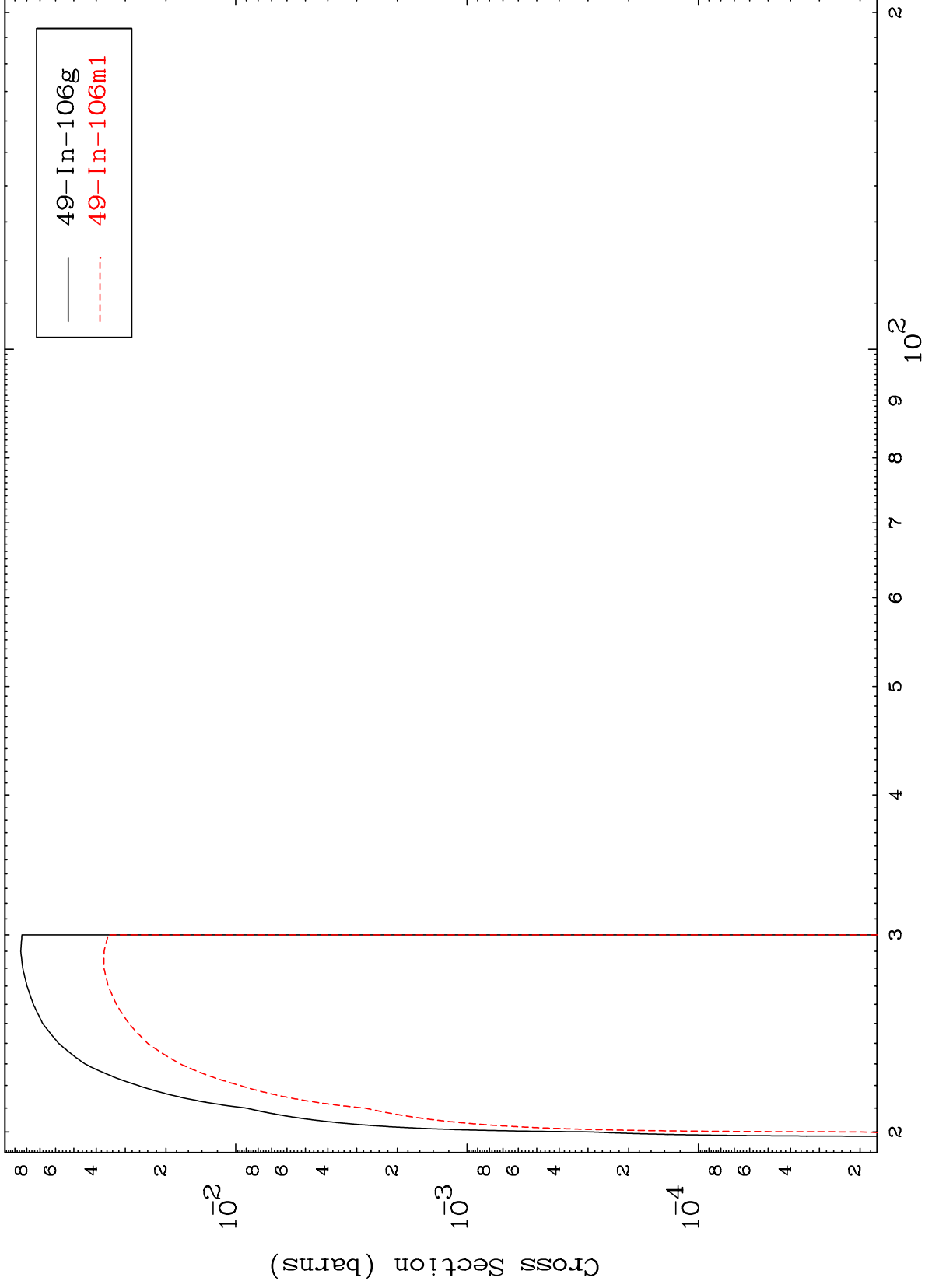
20

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(n,3n)

49-In-108

Radionuclide Production Cross Section



21

Incident Energy (MeV)

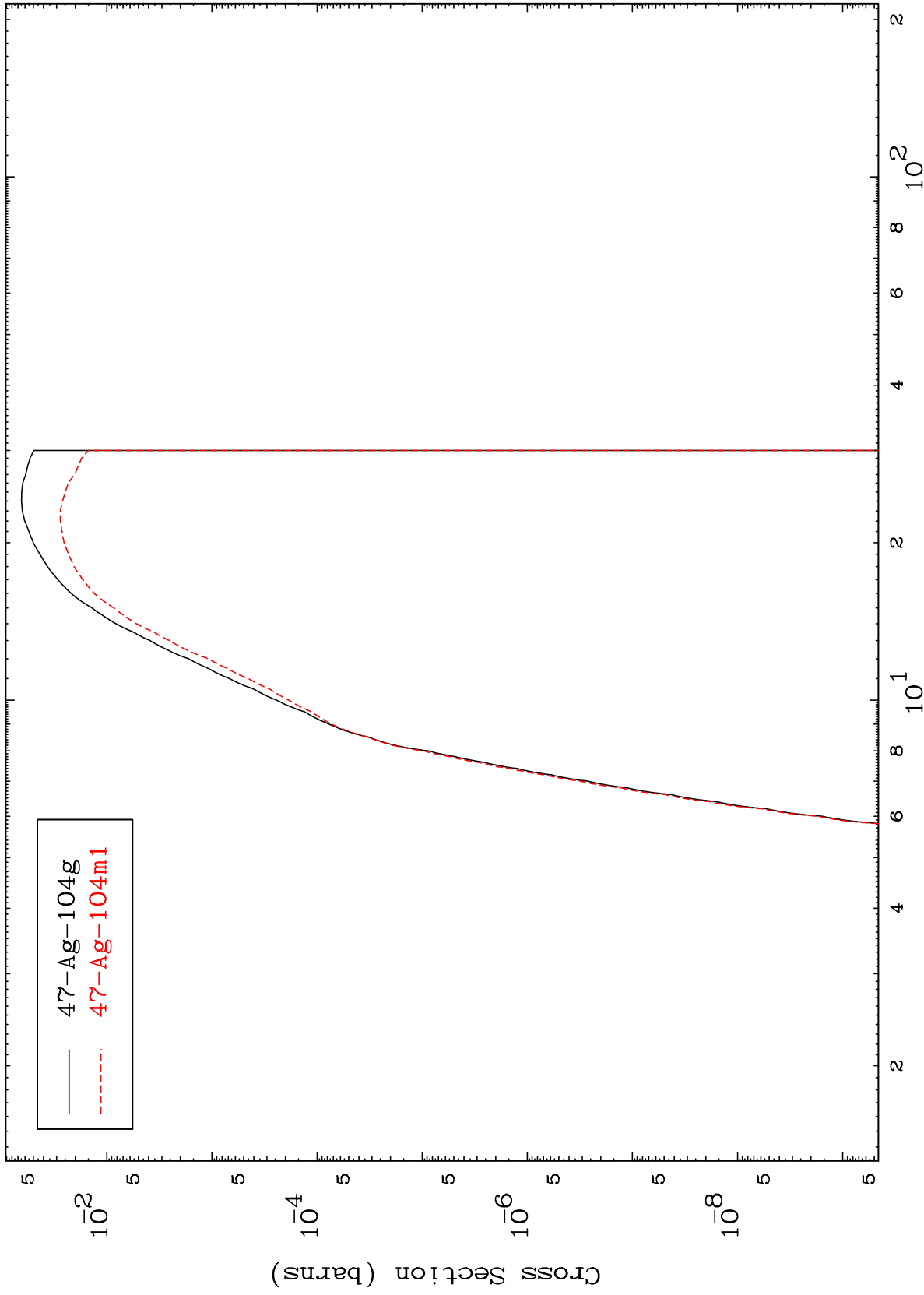
49-In-108

MAT 4911

(n,n')  $\alpha$

49-In-108

Radionuclide Production Cross Section



22

Incident Energy (MeV)

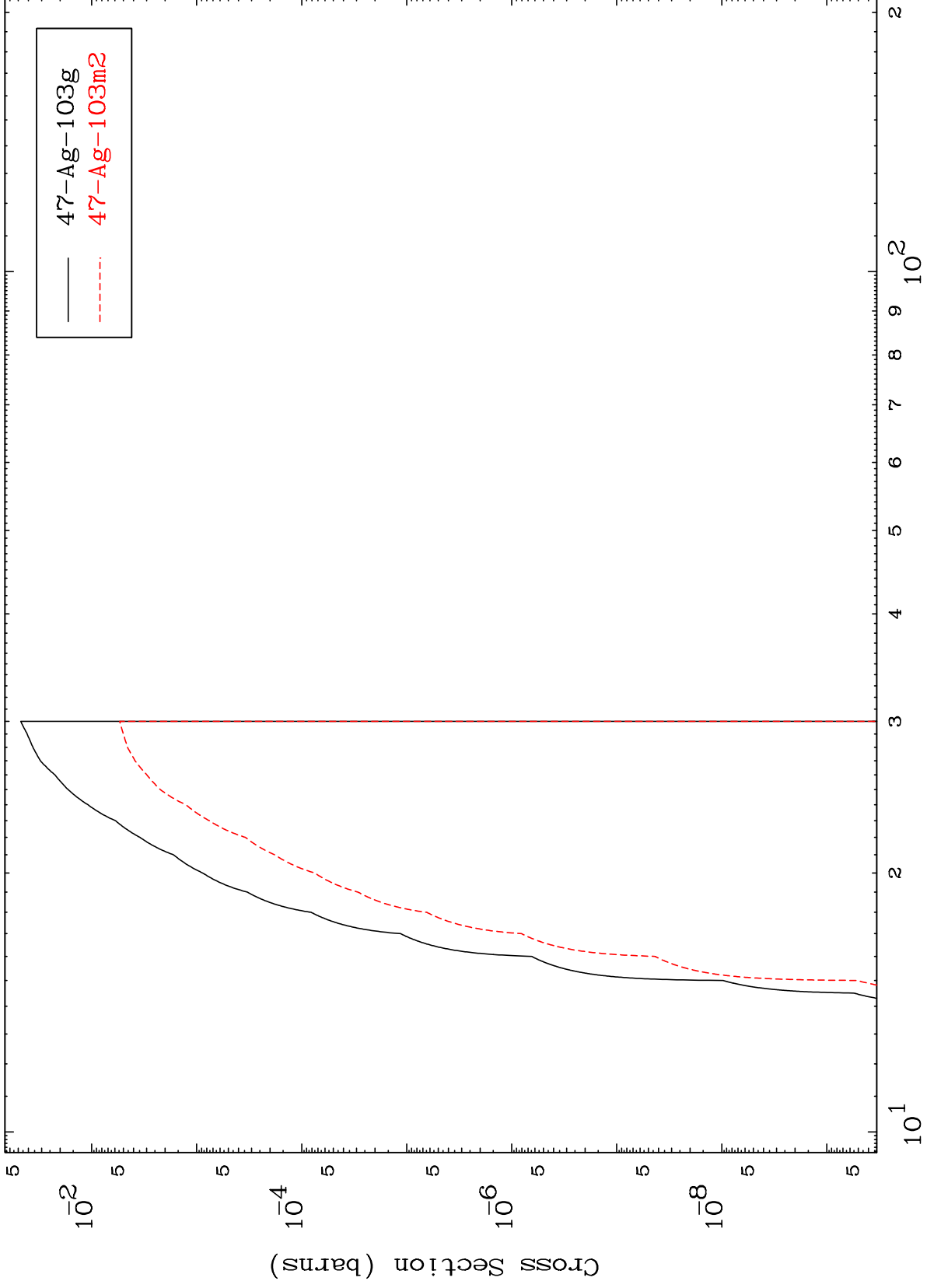
49-In-108

MAT 4911

(n,2n)  $\alpha$

49-In-108

Radionuclide Production Cross Section



23

Incident Energy (MeV)

49-In-108

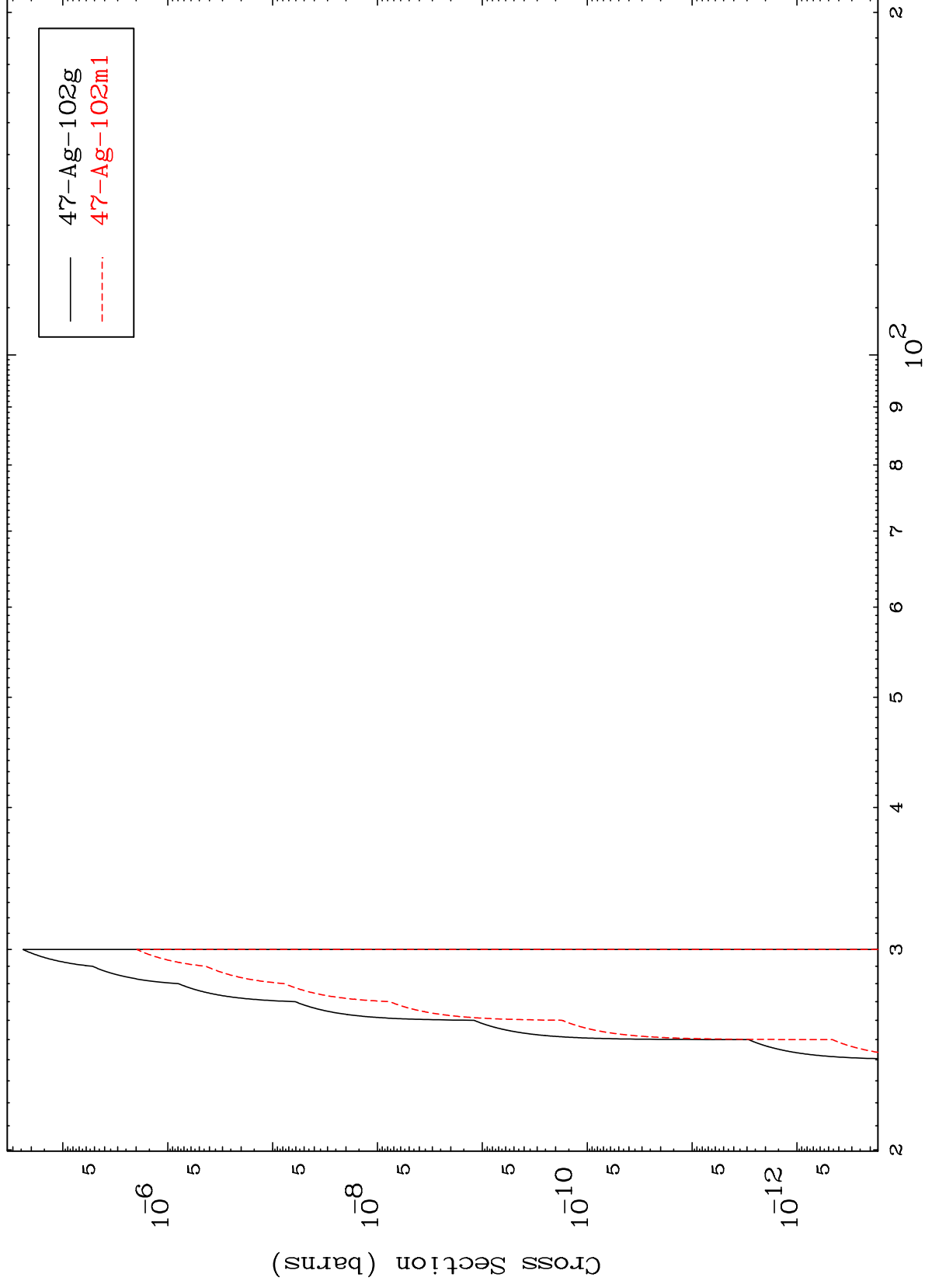


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(n,3n)  $\alpha$

49-In-108

Radionuclide Production Cross Section



24

Incident Energy (MeV)

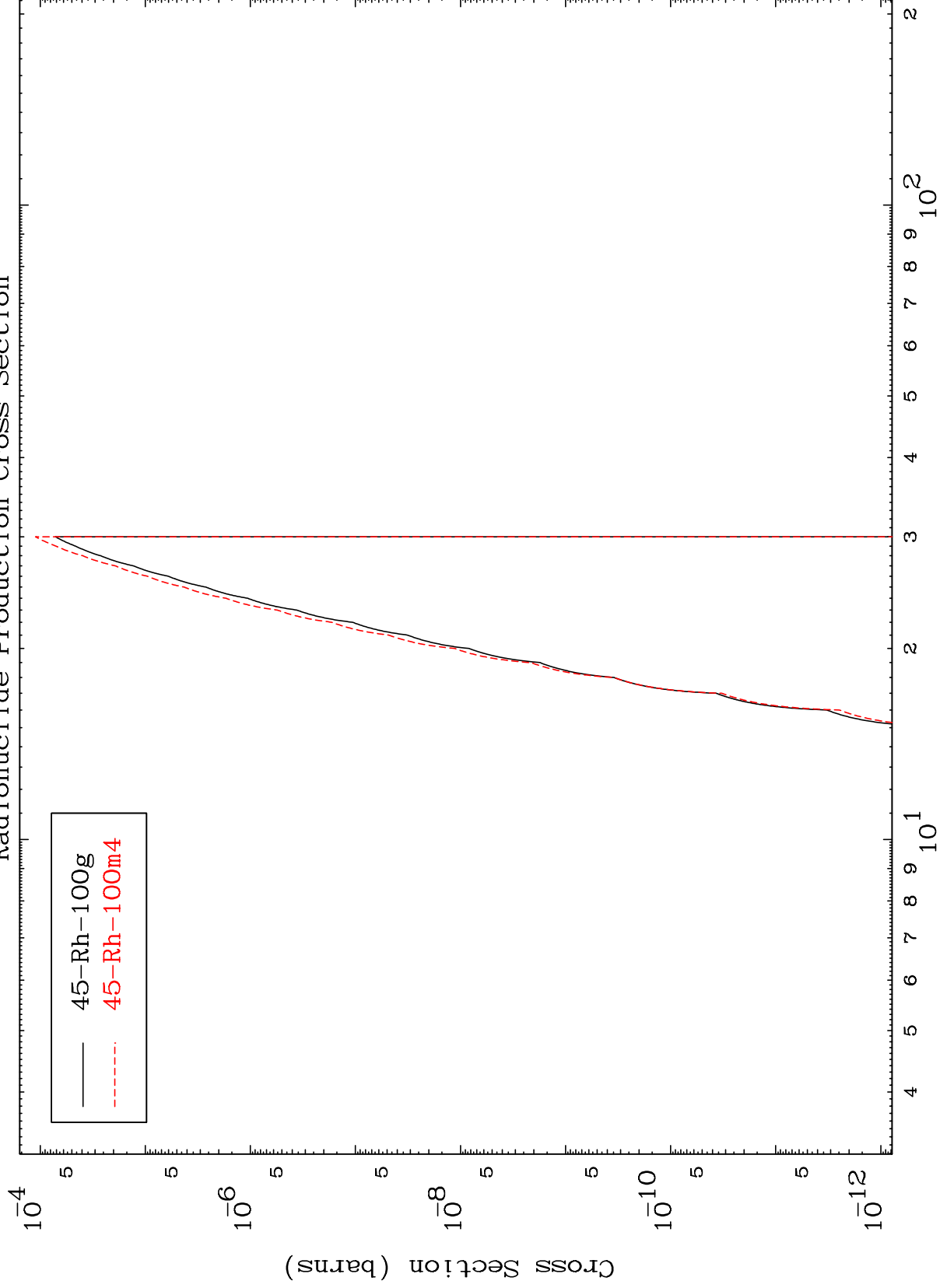
49-In-108

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(n,n') 2α

49-In-108

Radionuclide Production Cross Section



25

Incident Energy (MeV)

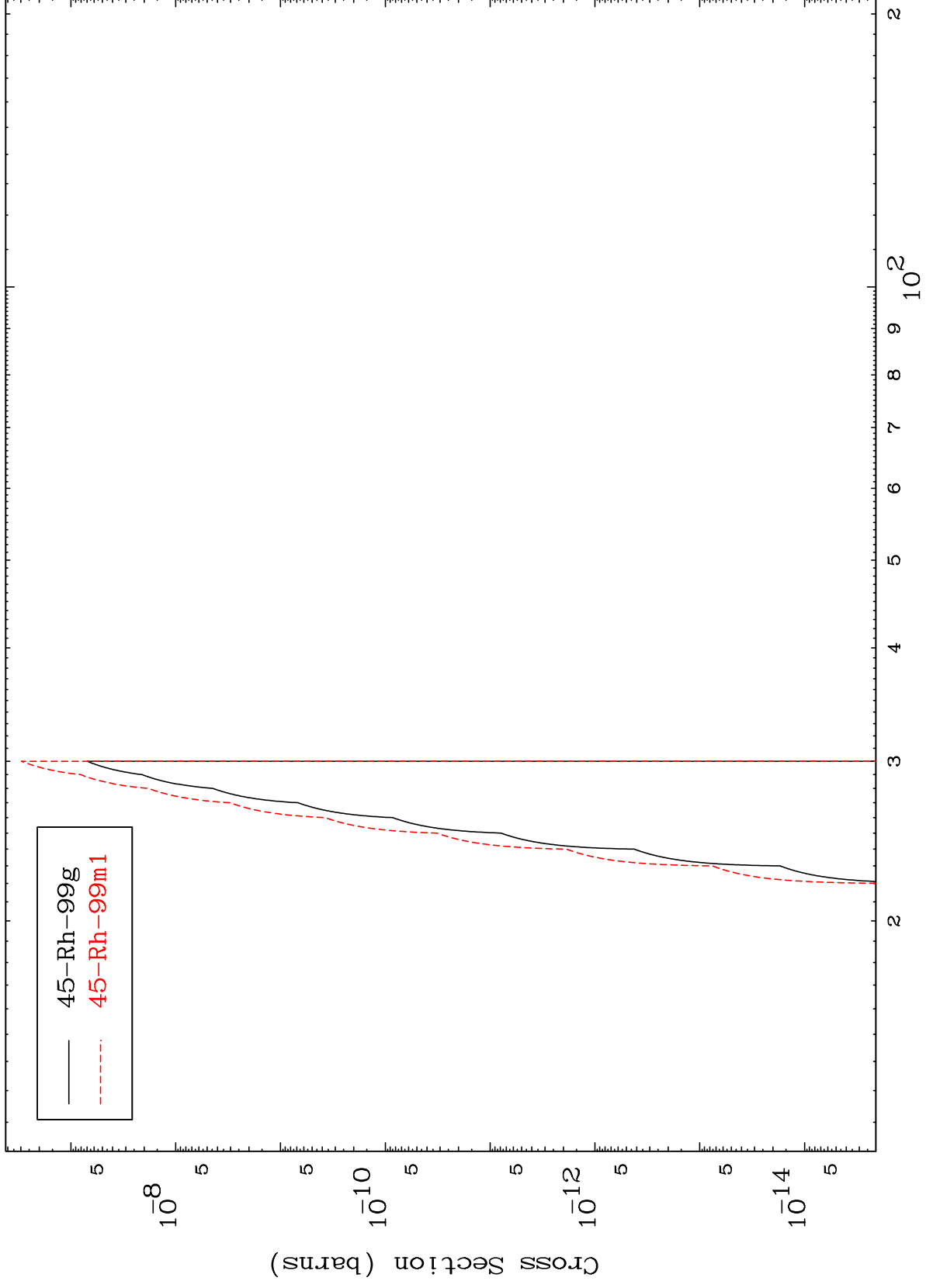
49-In-108

MAT 4911

(n,2n) 2 $\alpha$

49-In-108

Radionuclide Production Cross Section

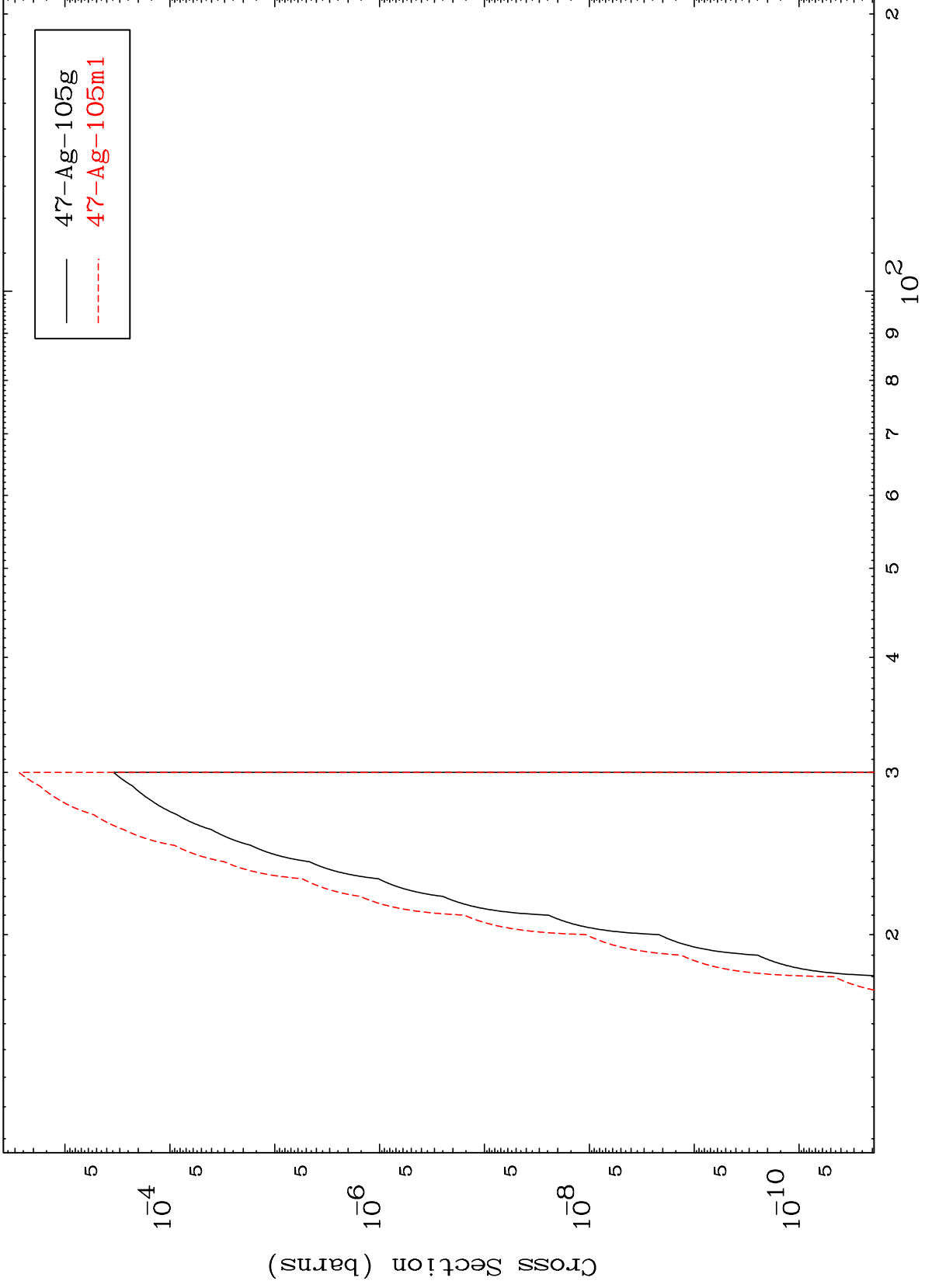


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Incident Energy (MeV)

49-In-108

Radionuclide Production Cross Section

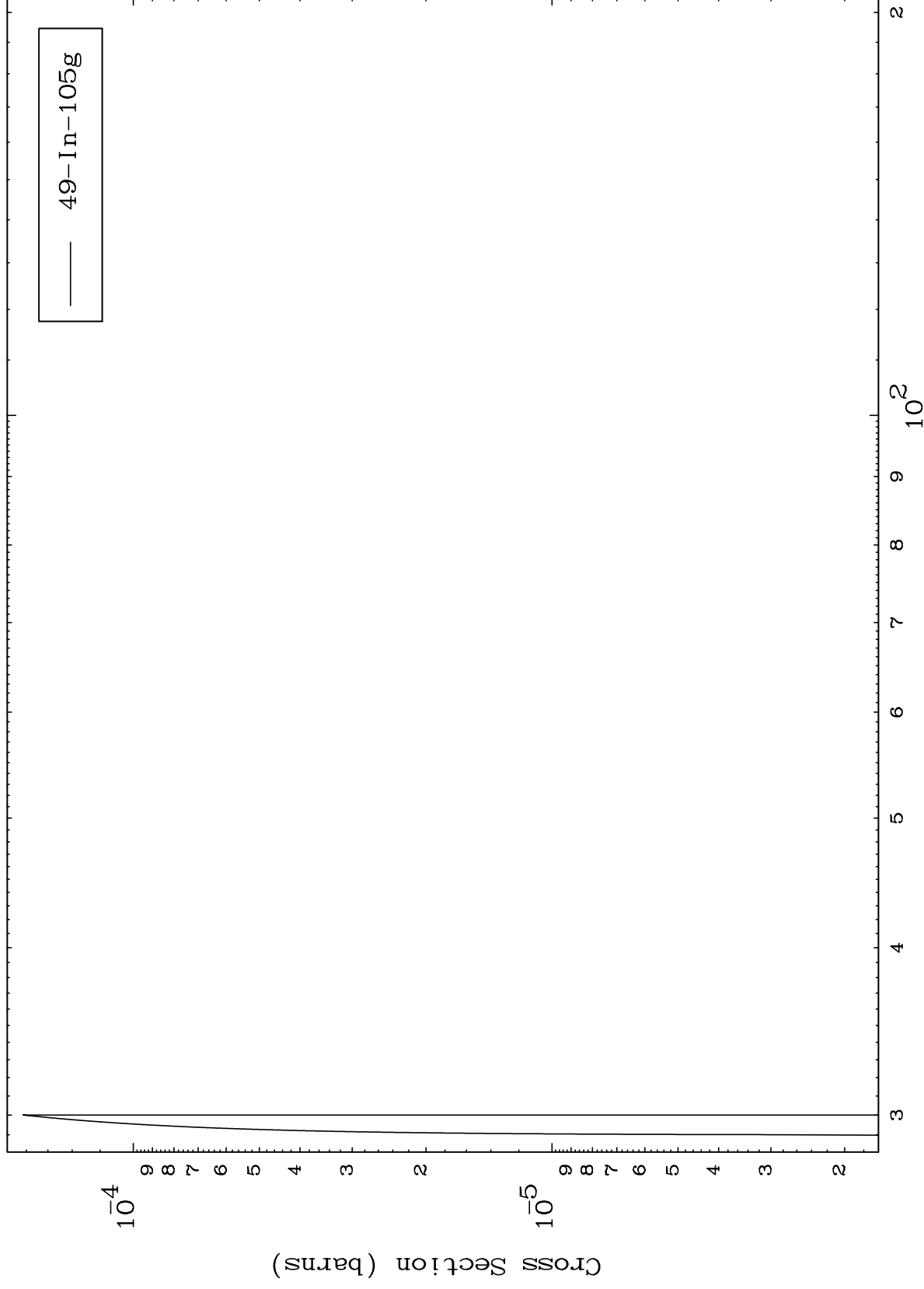


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(n,4n)

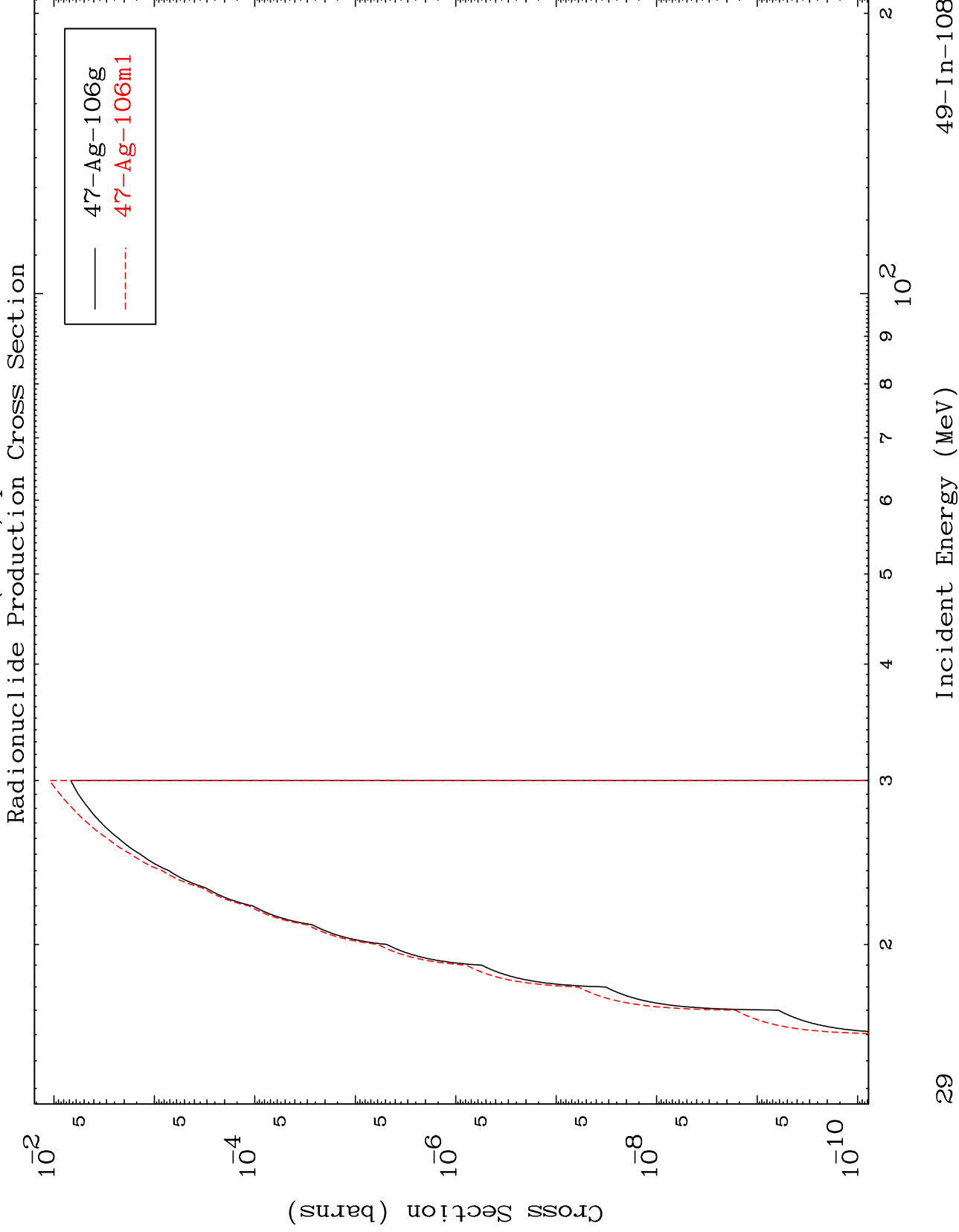
Radionuclide Production Cross Section



28

Incident Energy (MeV)

49-In-108

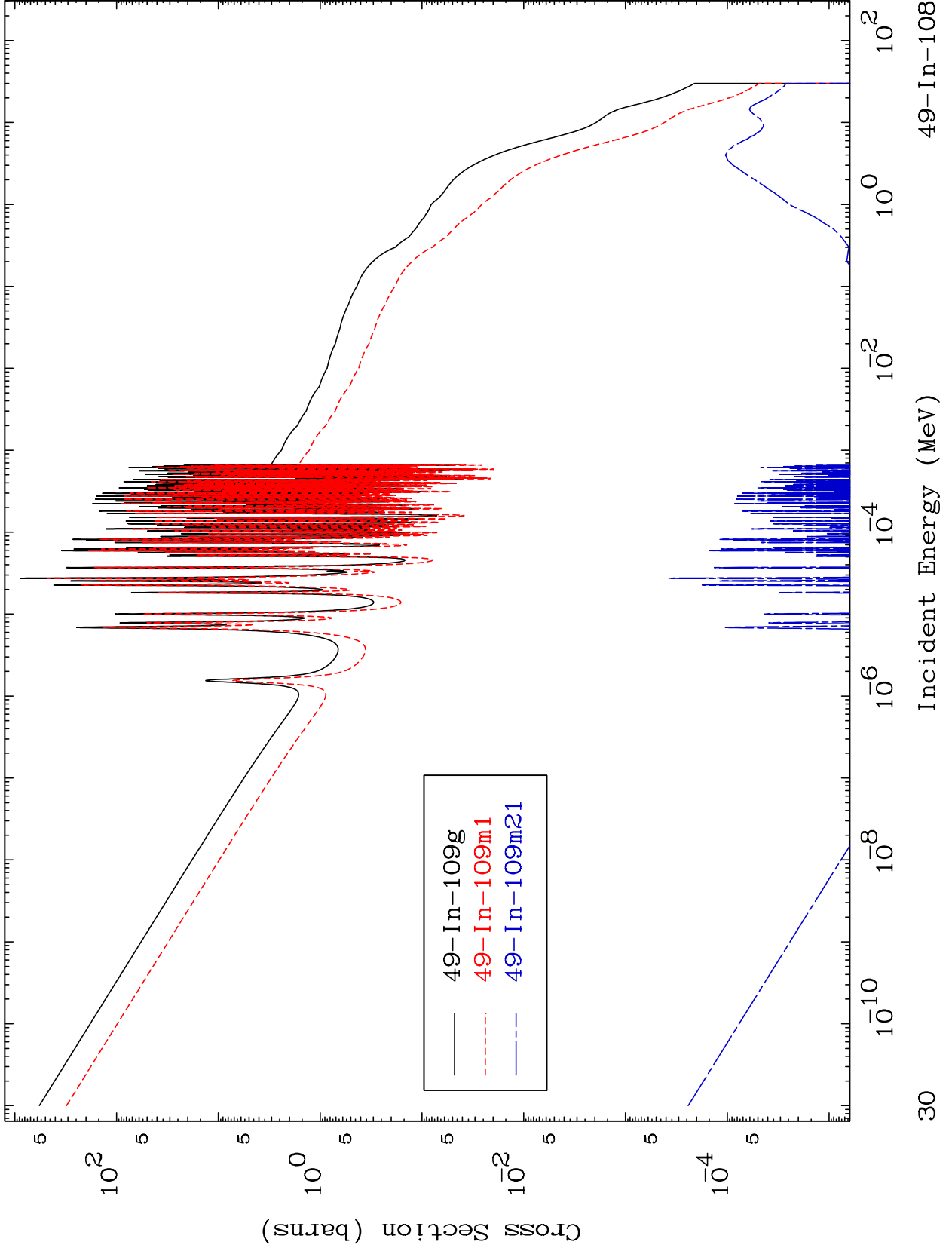


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49-In-108

Radionuclide Production Cross Section

(n,  $\gamma$ )

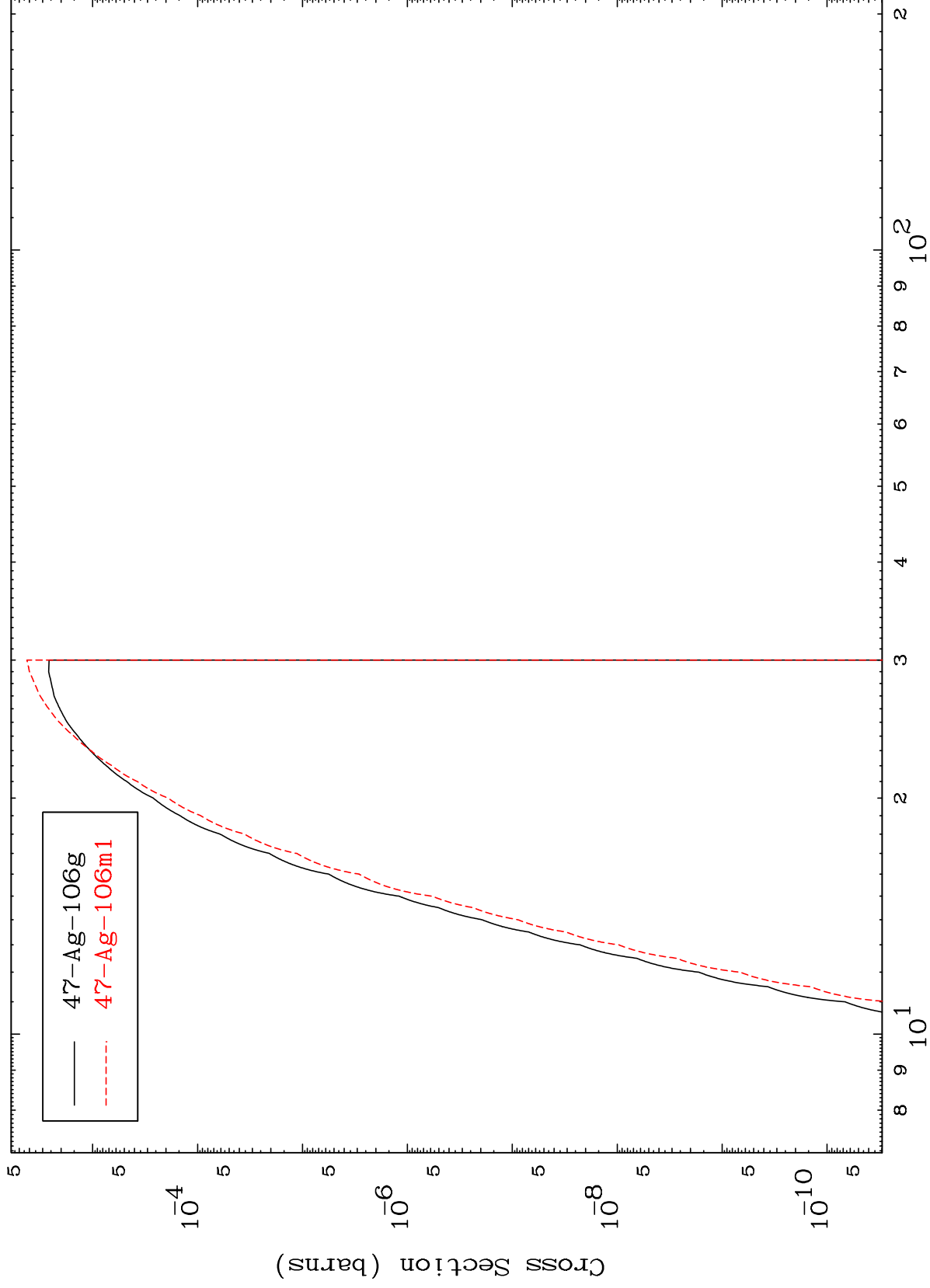


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(n,He-3)

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Radionuclide Production Cross Section



31

Incident Energy (MeV)

49-In-108

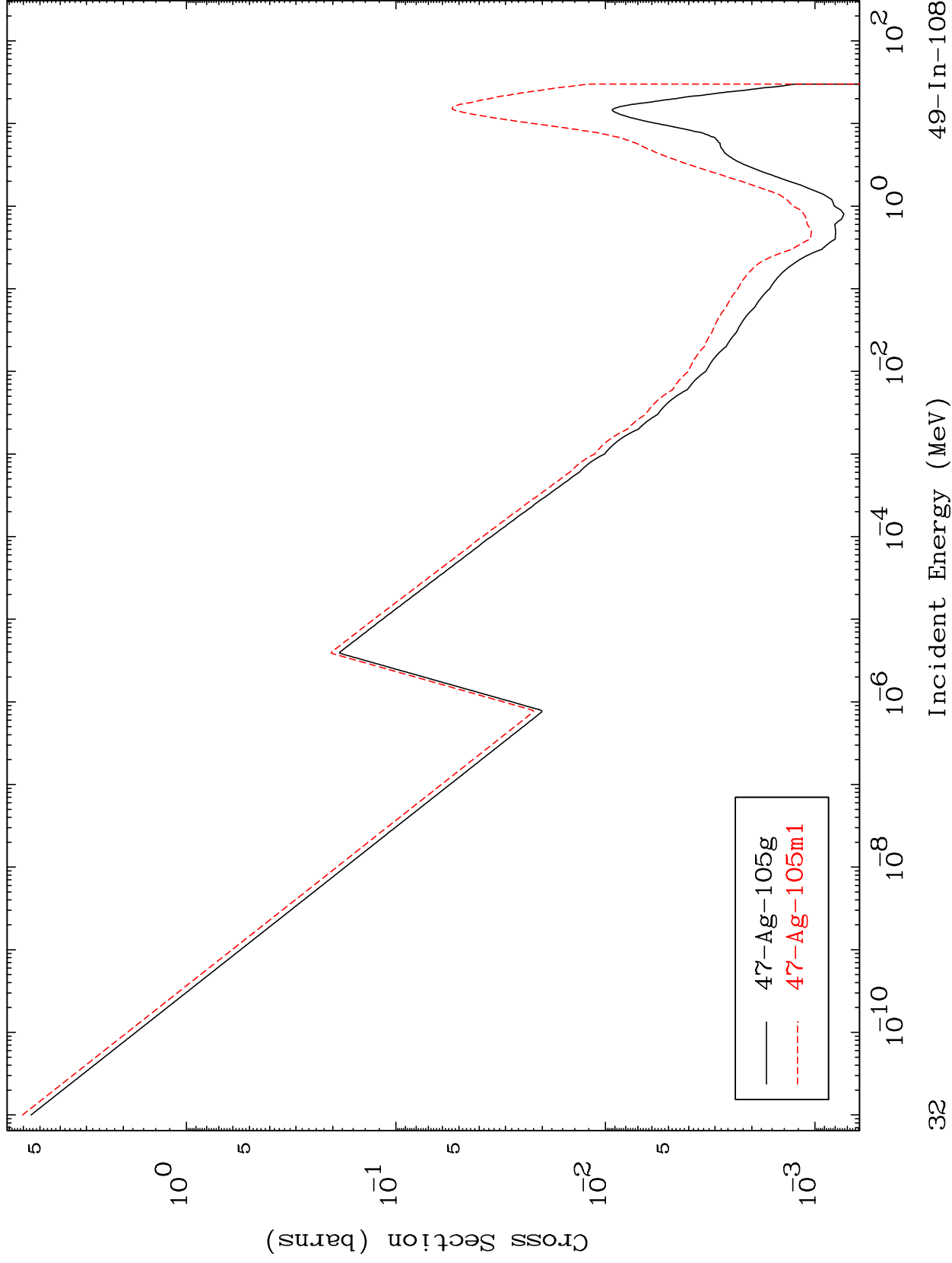


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49-In-108

Radionuclide Production Cross Section

(n,  $\alpha$ )

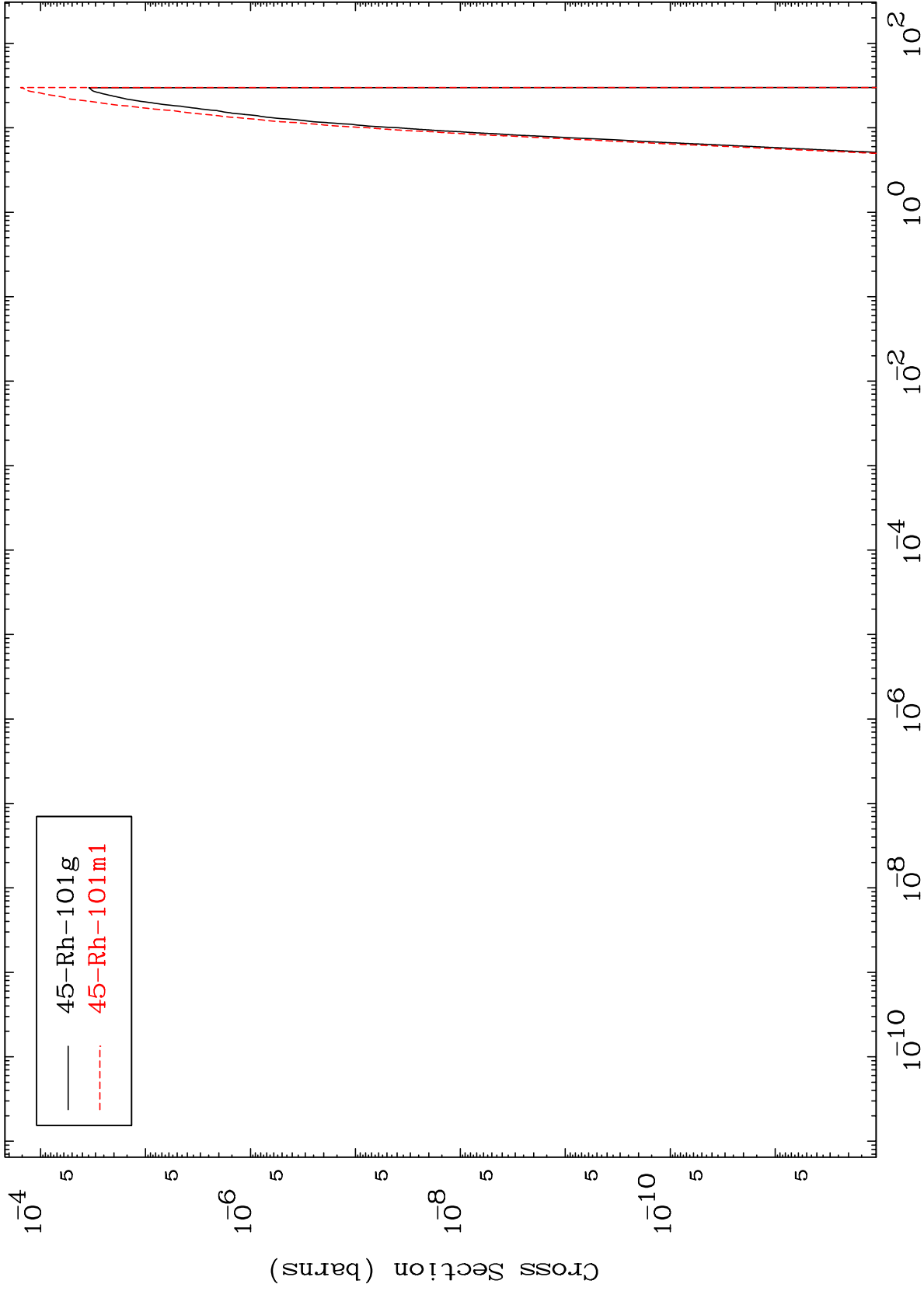


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(n,2α)

49-In-108

Radionuclide Production Cross Section



33

Incident Energy (MeV)

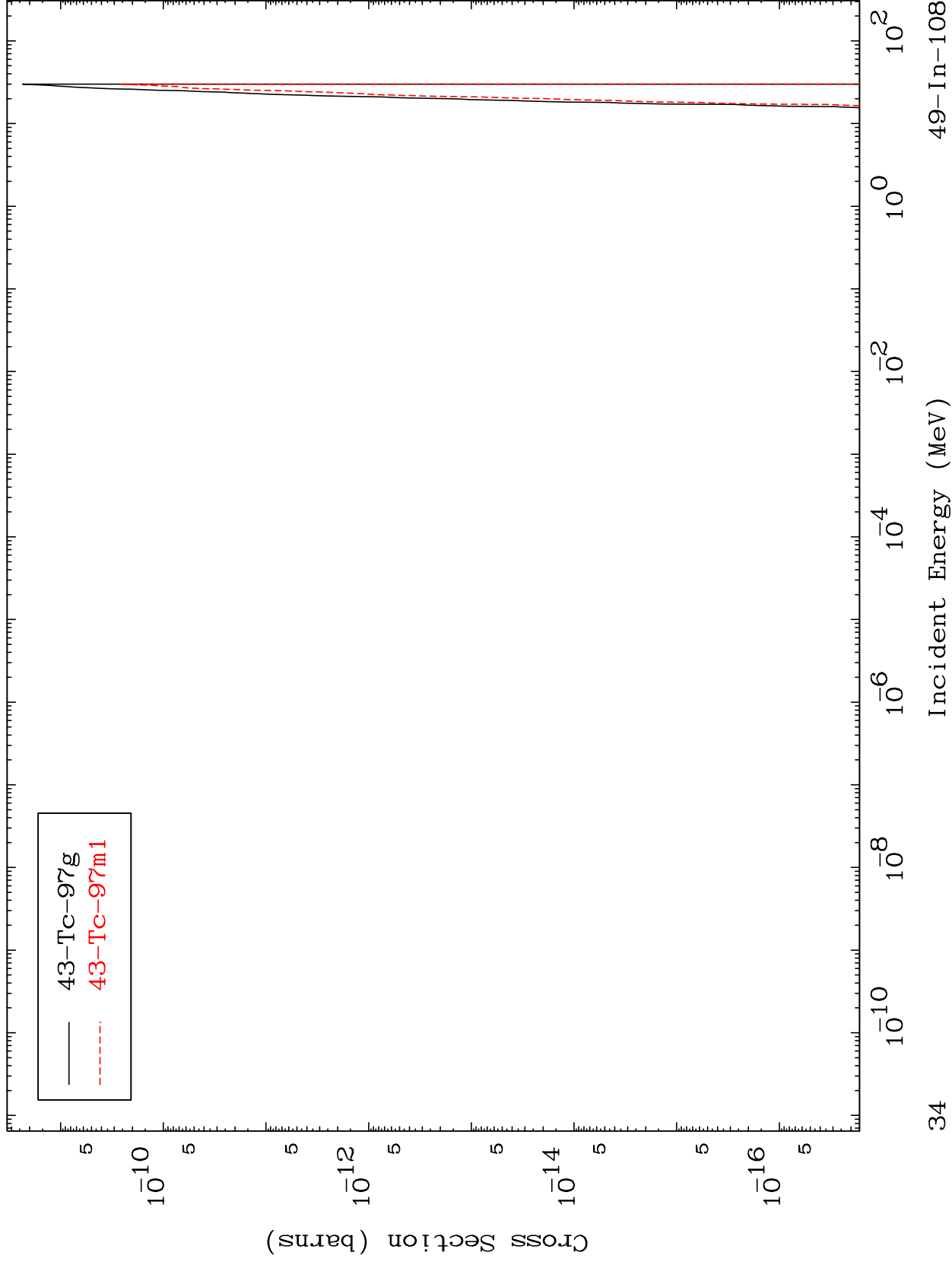
49-In-108

MAT 4911

(n,3 $\alpha$ )

49-In-108

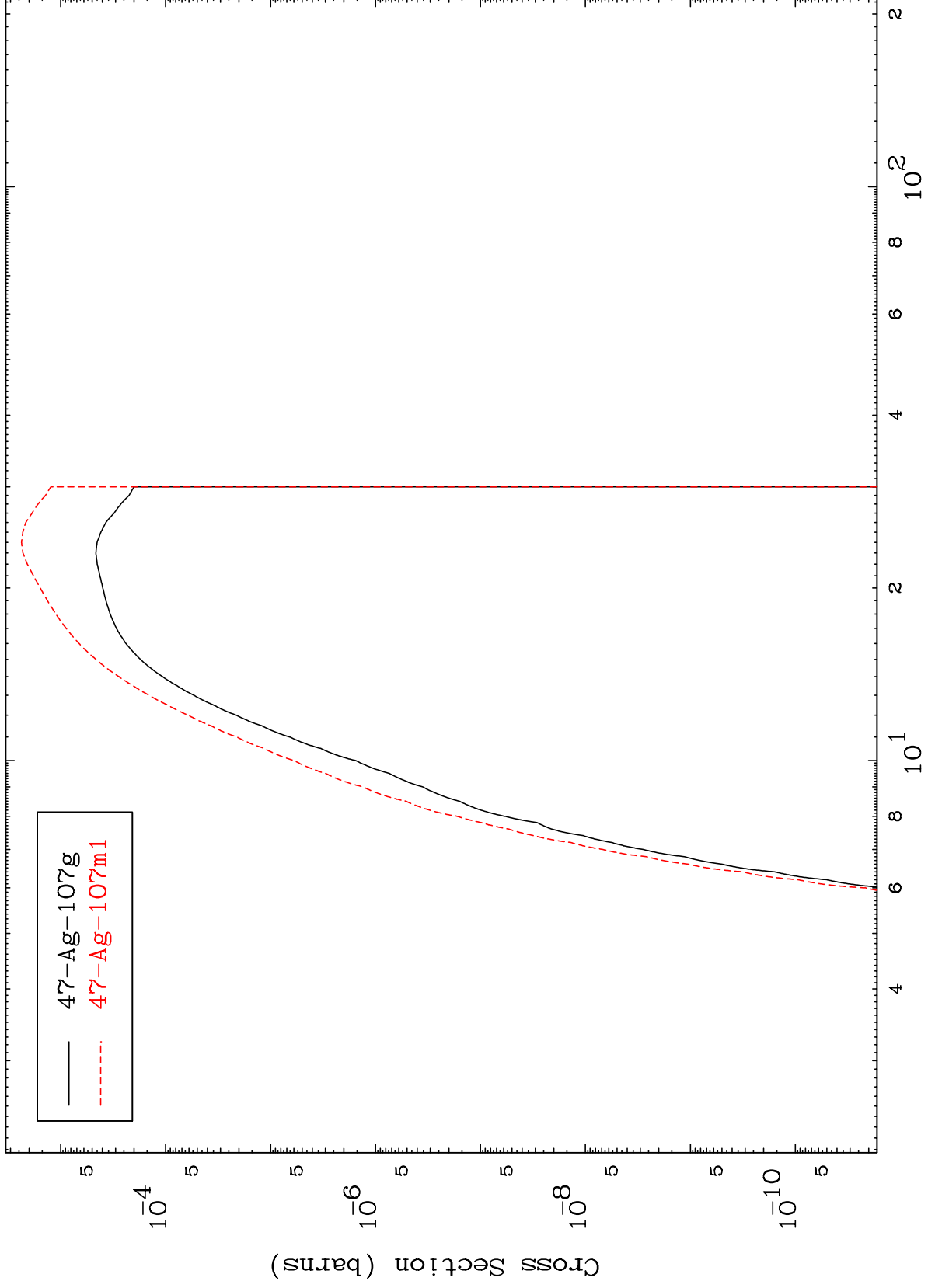
Radionuclide Production Cross Section



MAT 4911

49-In-108

(n,2p)  
Radionuclide Production Cross Section



35

49-In-108

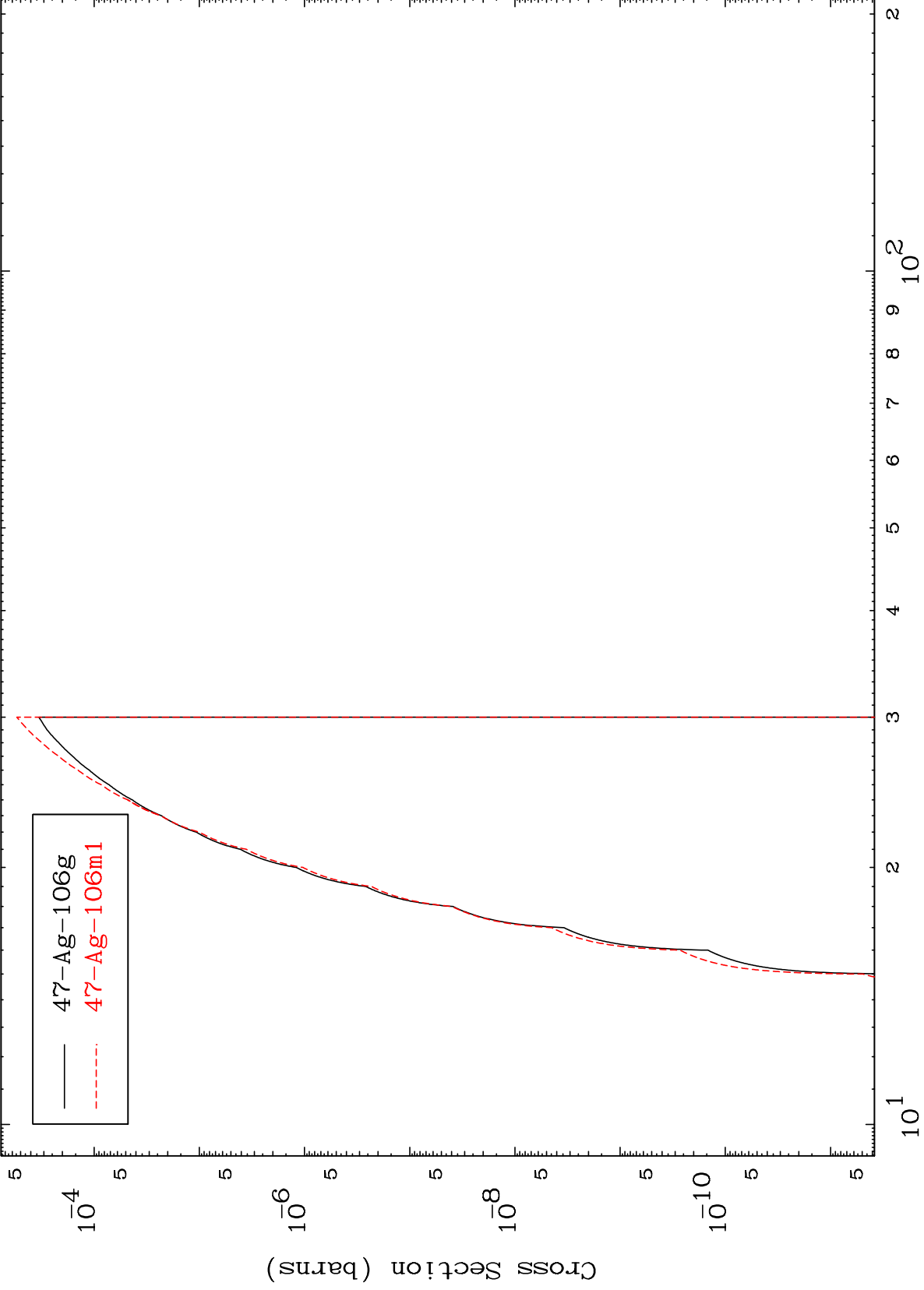
Incident Energy (MeV)

MAT 4911

(n,p) d

49-In-108

Radionuclide Production Cross Section



36

Incident Energy (MeV)

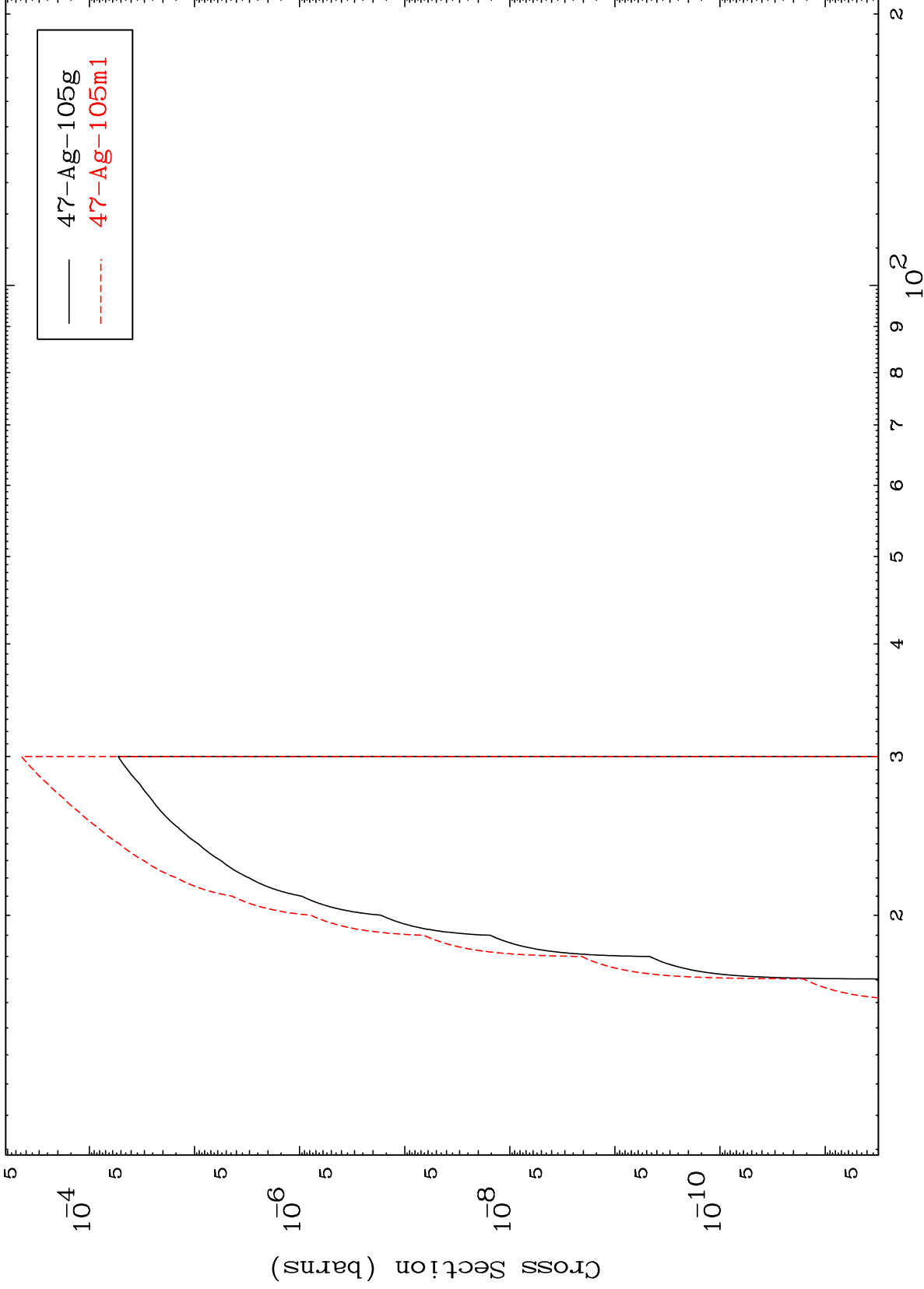
49-In-108

MAT 4911

(n,p) t

49-In-108

Radionuclide Production Cross Section



37

Incident Energy (MeV)

49-In-108