

Program EVALPLOT  
(Version 2015-2)

by

Dermott E. Cullen  
(Present Contact Information)

Dermott E. Cullen  
1466 Hudson Way  
Livermore, CA 94550  
U.S.A.

Tele: 925-443-1911

E.Mail:redcullen1@comcast.net  
Web:home.comcast.net/~redcullen1

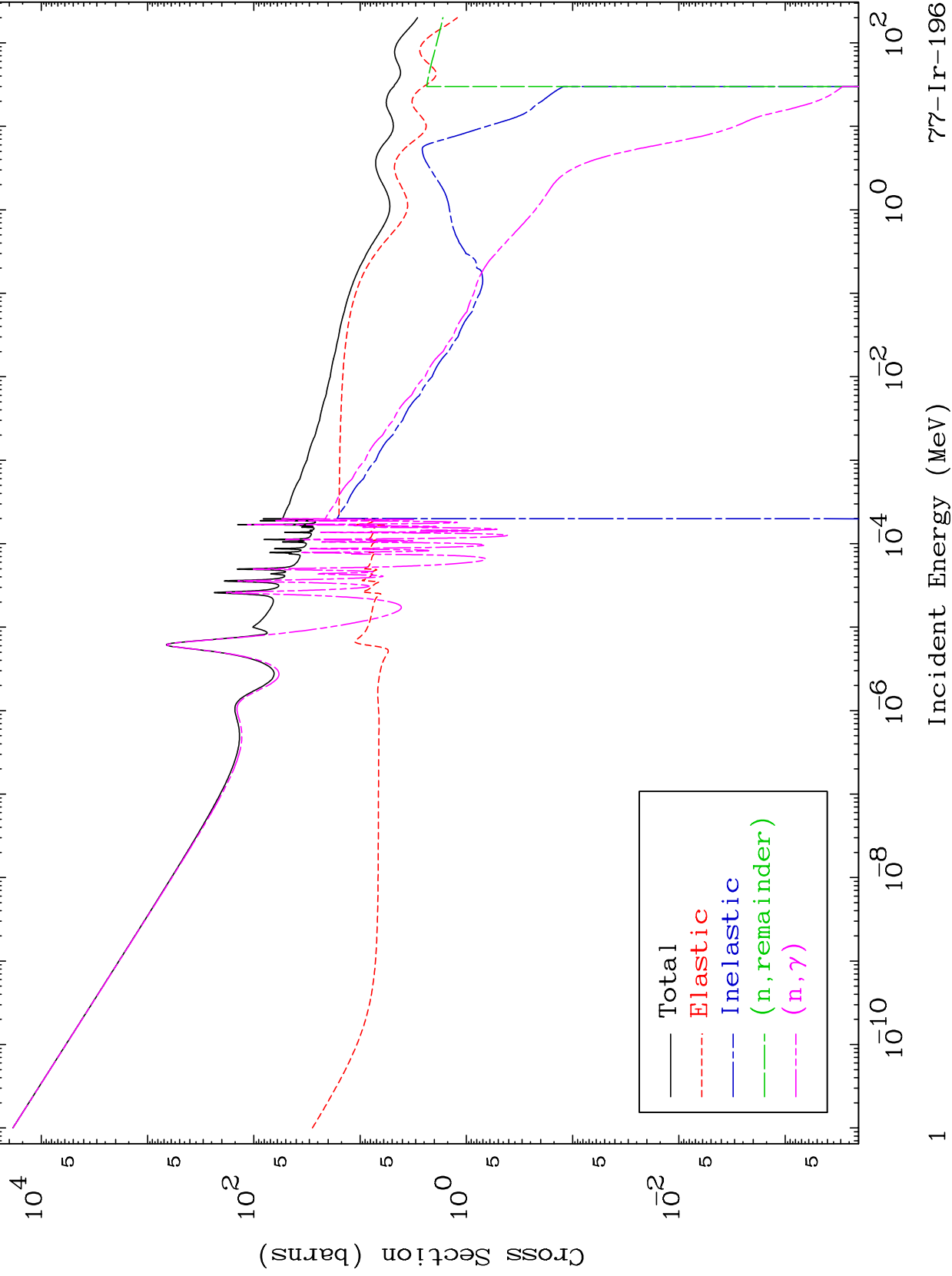
Press Mouse Button to Start

MAT 7741

Major

293 Kelvin Cross Sections

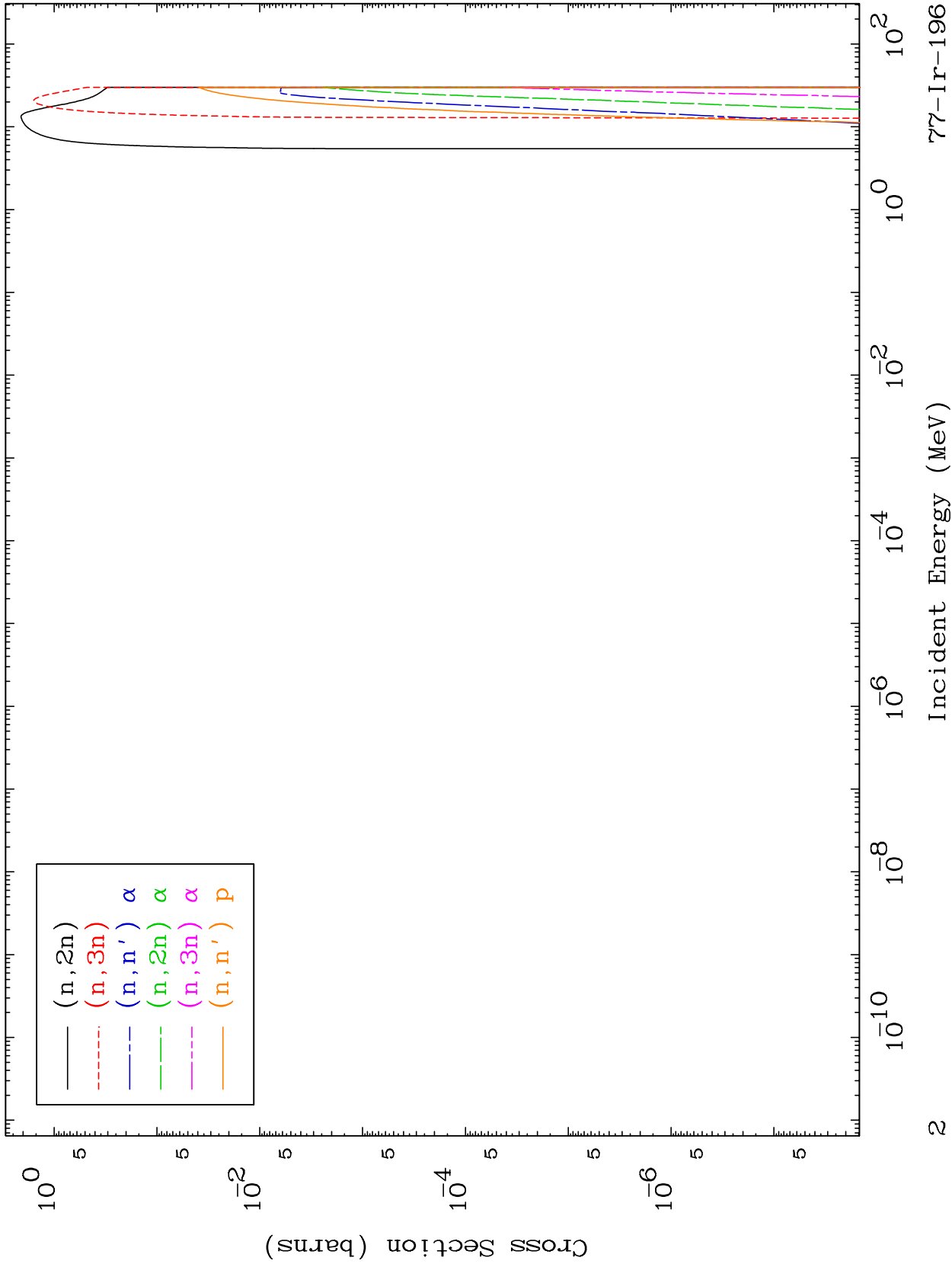
77-Ir-196



MAT 7741

Neutron Production  
293 Kelvin Cross Sections

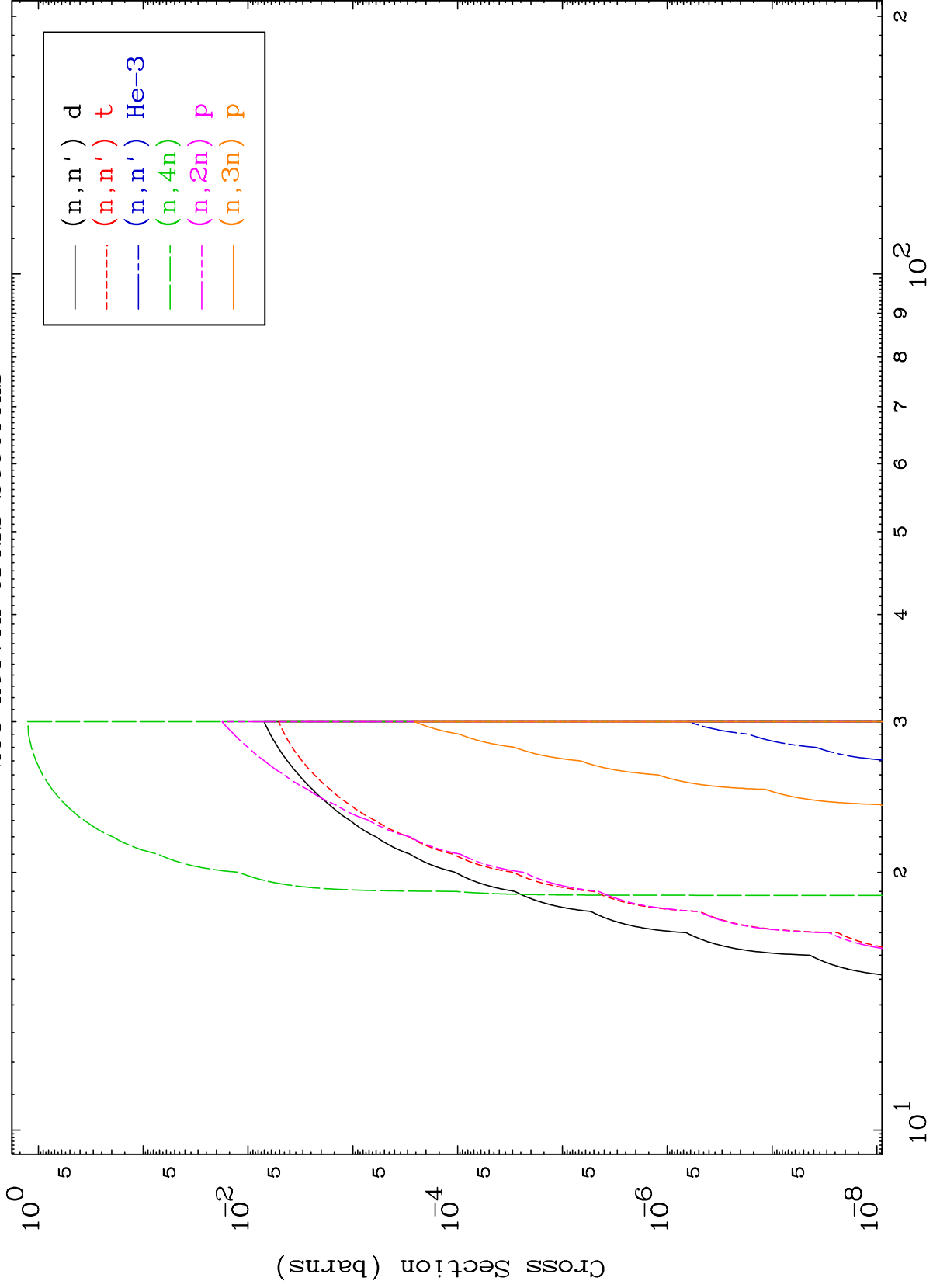
77-Ir-196



MAT 7741

Neutron Production  
293 Kelvin Cross Sections

77-Ir-196



77-Ir-196

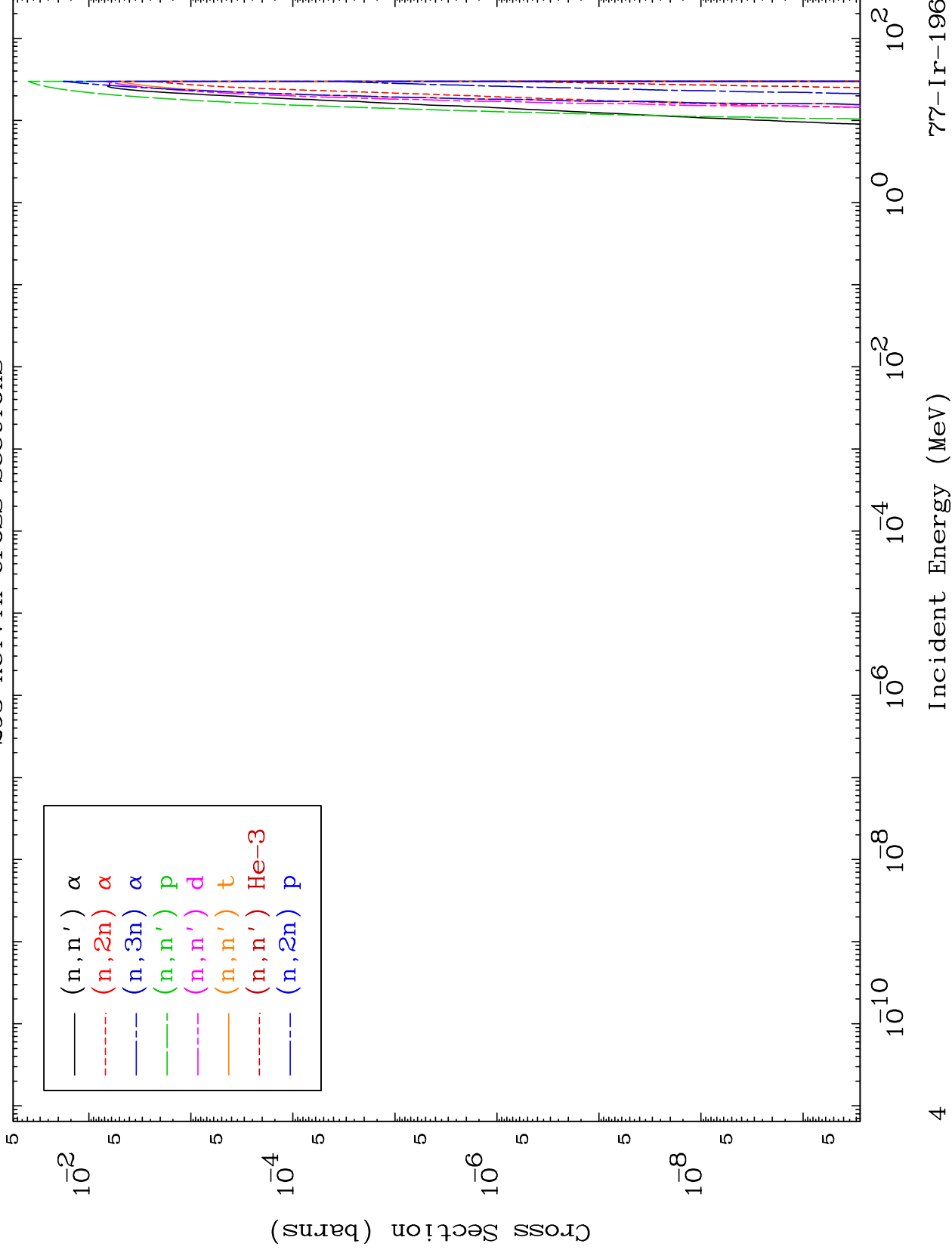
Incident Energy (MeV)

3

MAT 7741

Charged Particle  
293 Kelvin Cross Sections

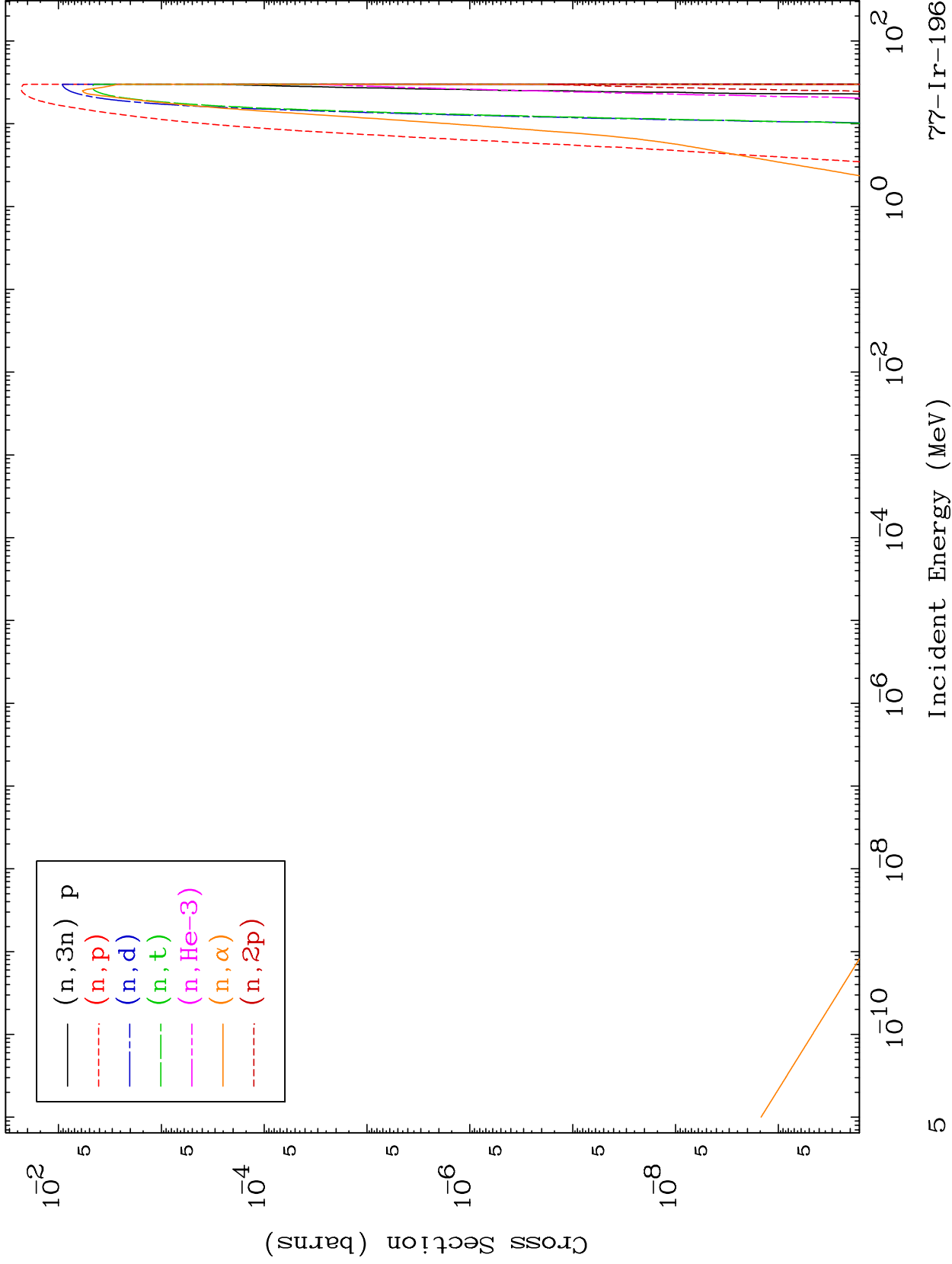
77-Ir-196

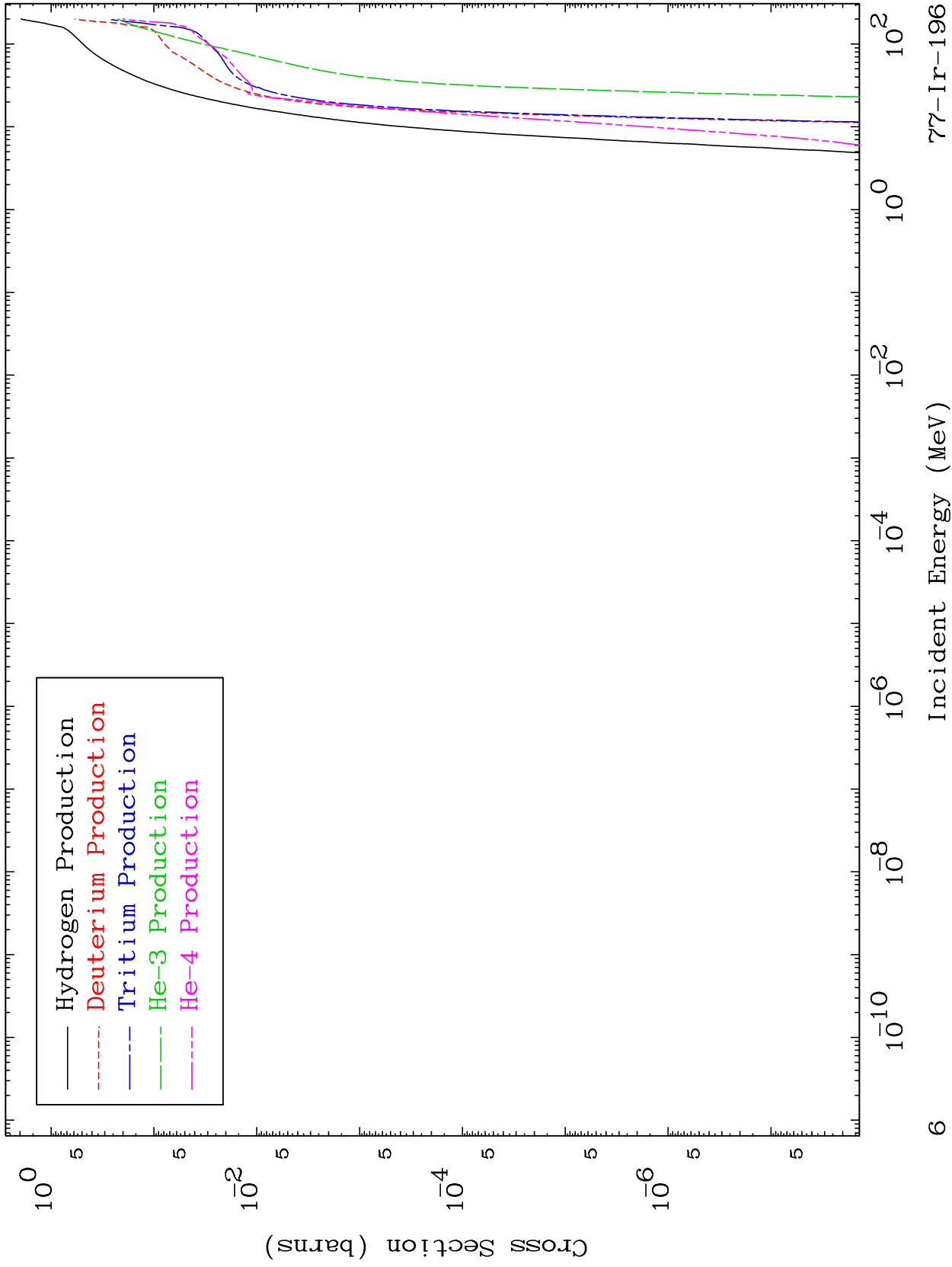


MAT 7741

Charged Particle  
293 Kelvin Cross Sections

77-Ir-196

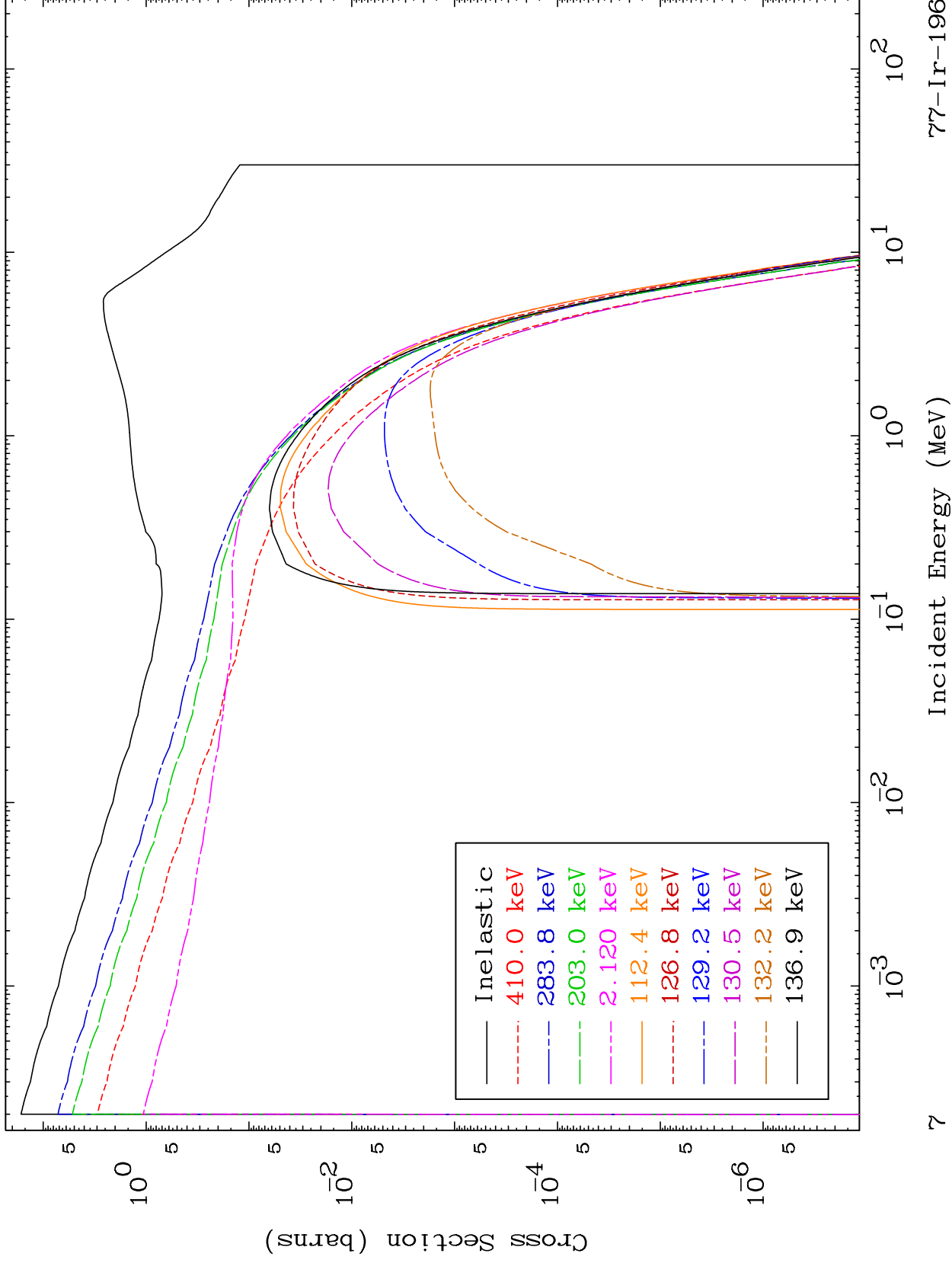




MAT 7741

(n,n') Level  
293 Kelvin Cross Sections

77-Ir-196

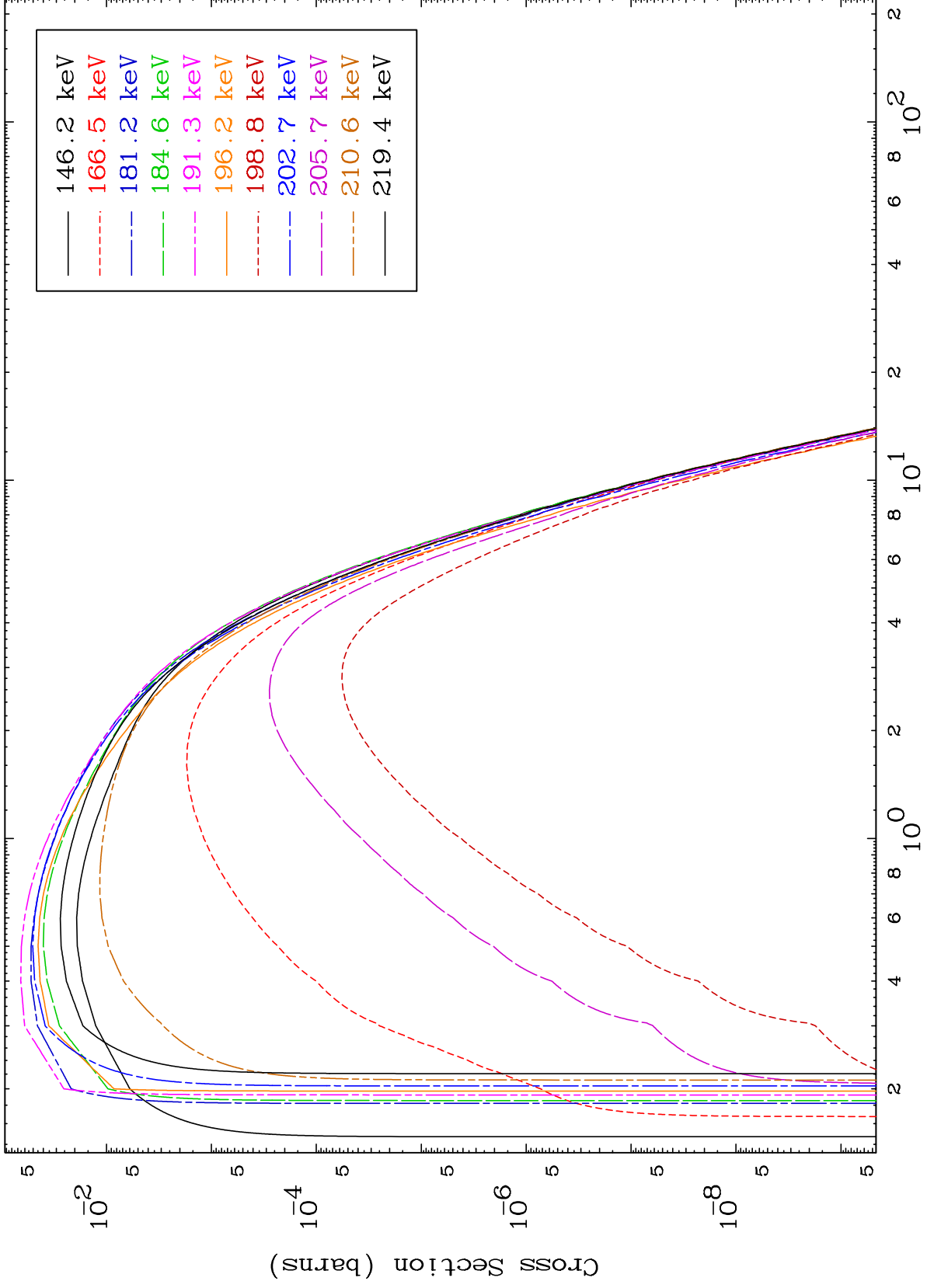




MAT 7741

(n,n') Level  
293 Kelvin Cross Sections

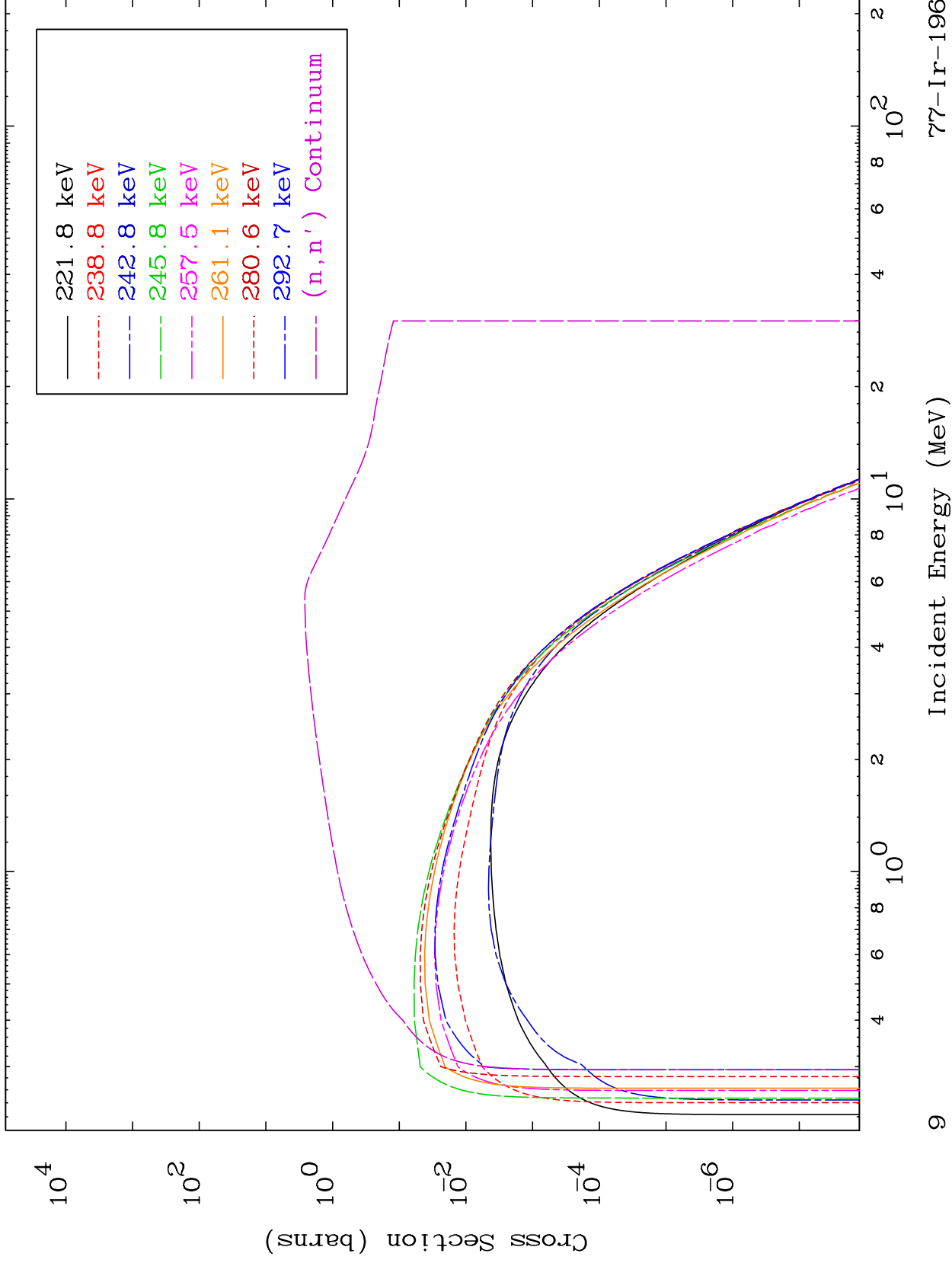
77-Ir-196



8

Incident Energy (MeV)

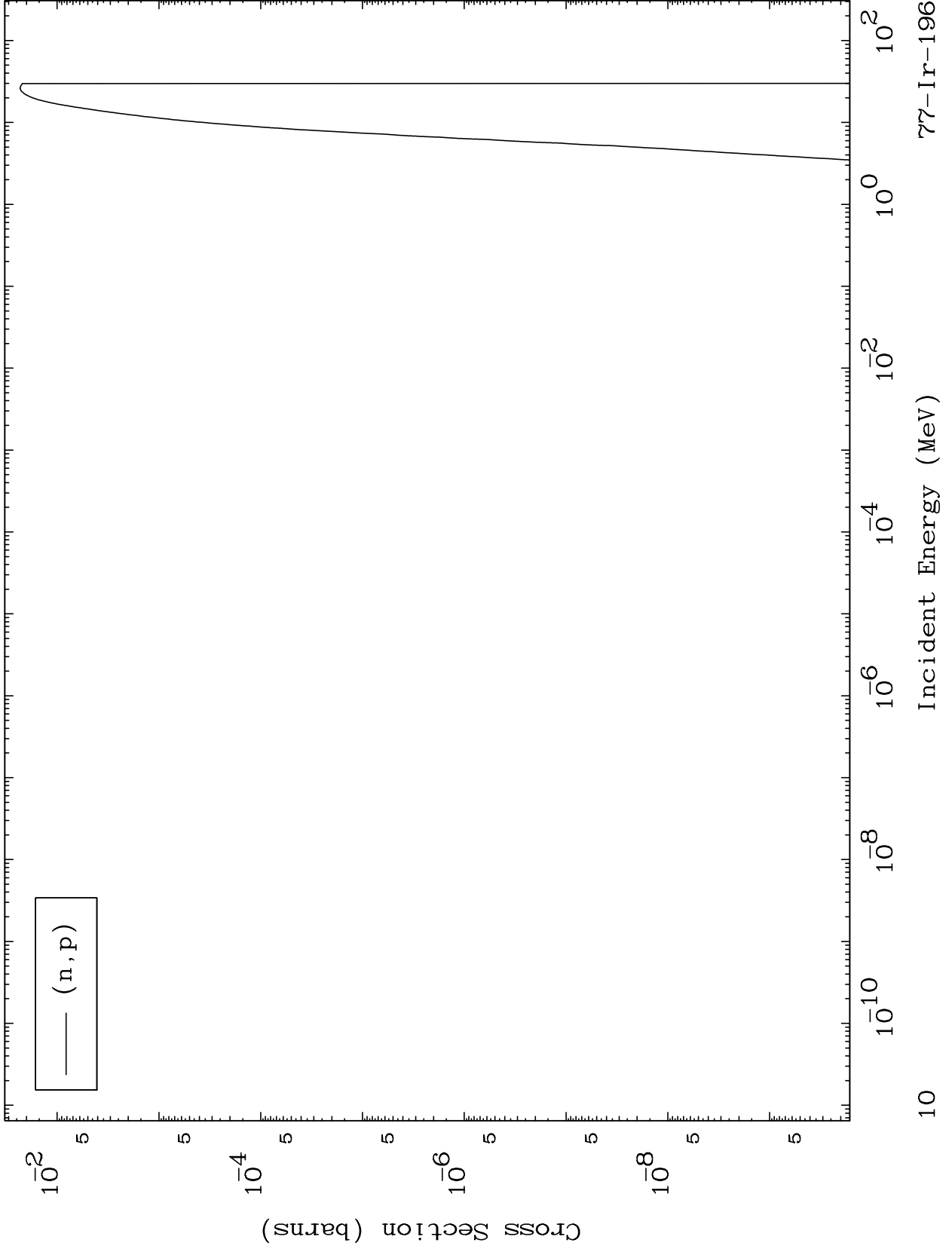
77-Ir-196



MAT 7741

(n,p) Levels  
293 Kelvin Cross Sections

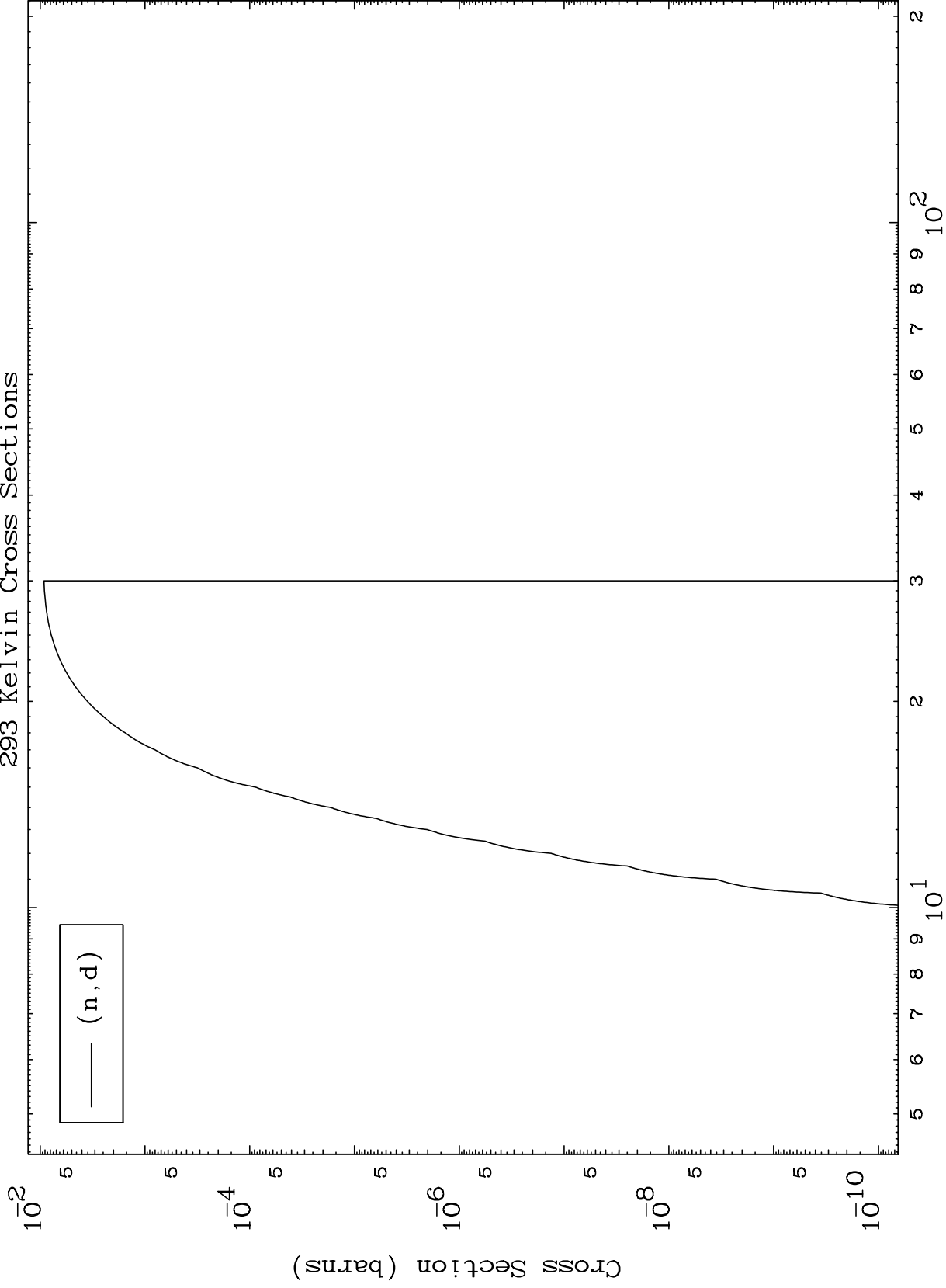
77-Ir-196



MAT 7741

(n,d) Levels  
293 Kelvin Cross Sections

77-Ir-196



11

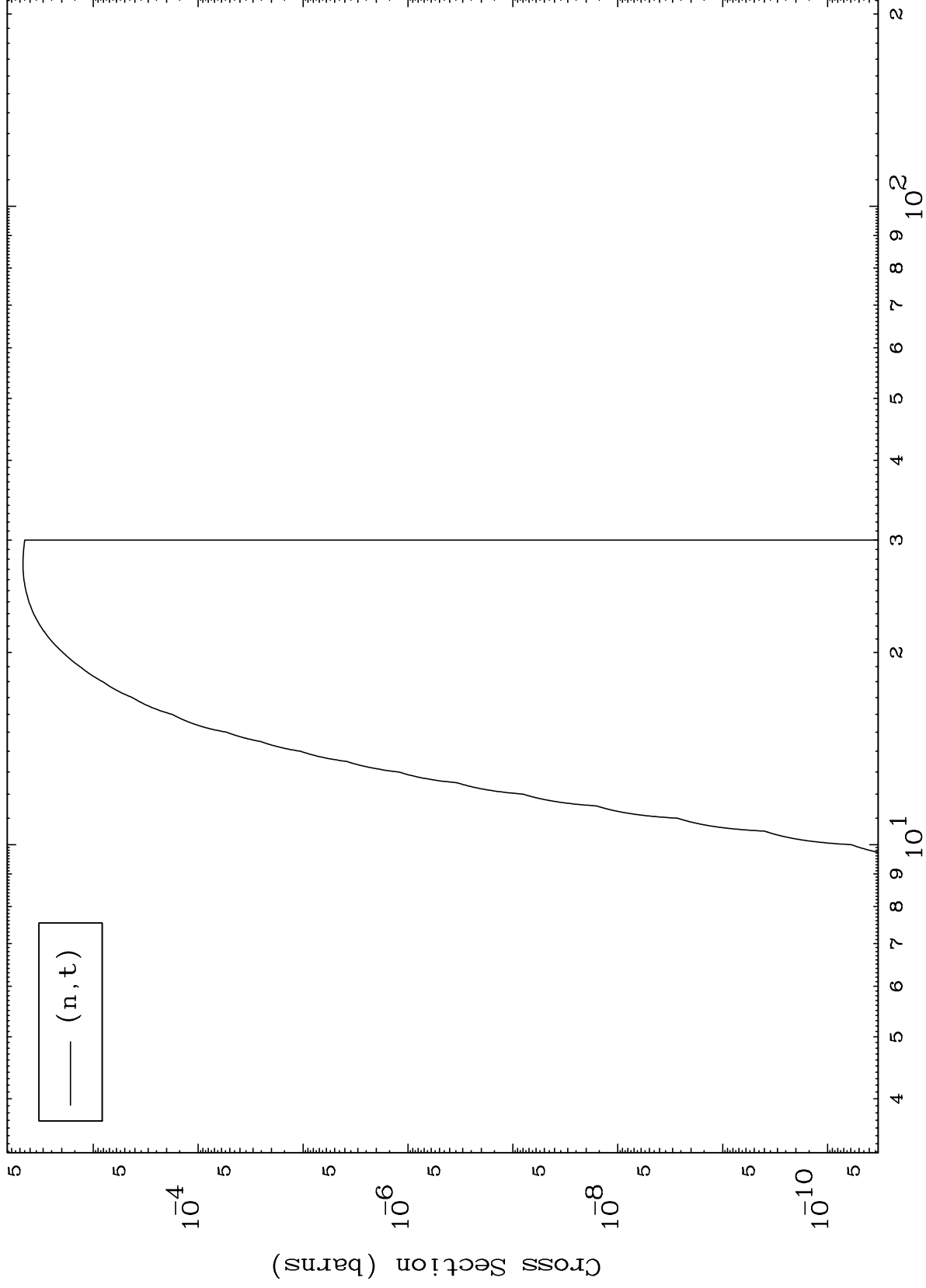
Incident Energy (MeV)

77-Ir-196

MAT 7741

(n,t) Levels  
293 Kelvin Cross Sections

77-Ir-196



12

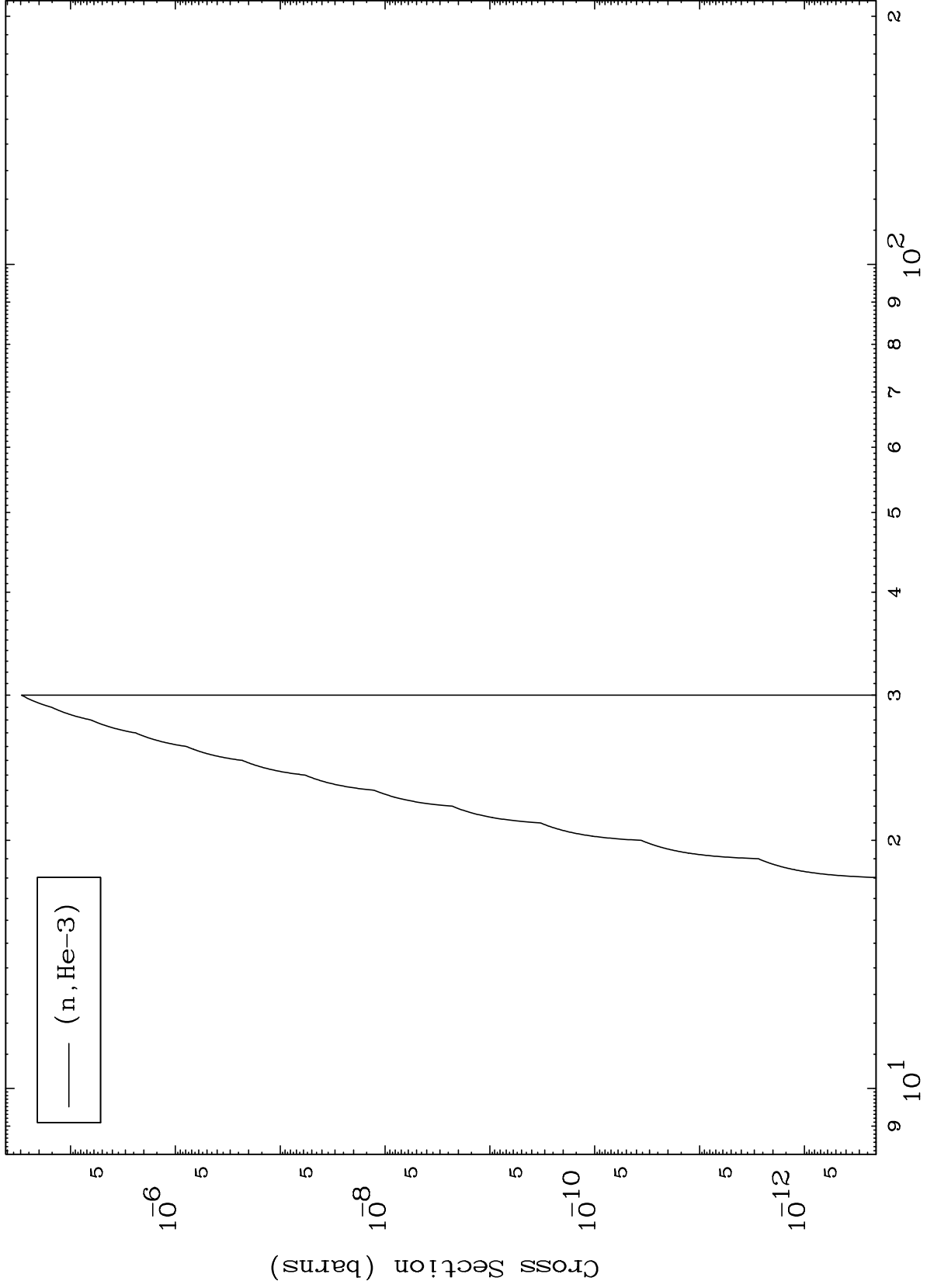
Incident Energy (MeV)

77-Ir-196

MAT 7741

(n,He3) Levels  
293 Kelvin Cross Sections

77-Ir-196



13

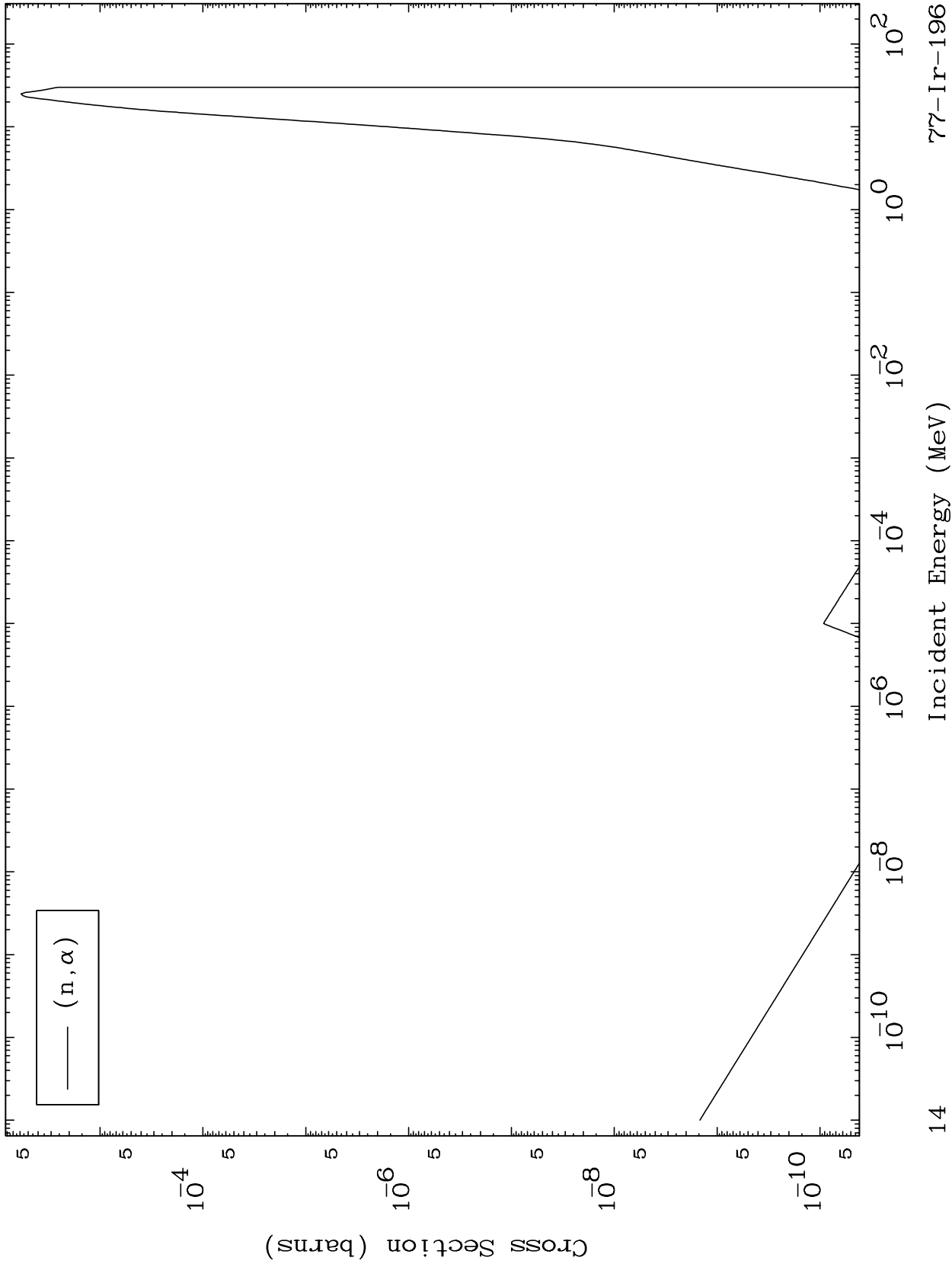
Incident Energy (MeV)

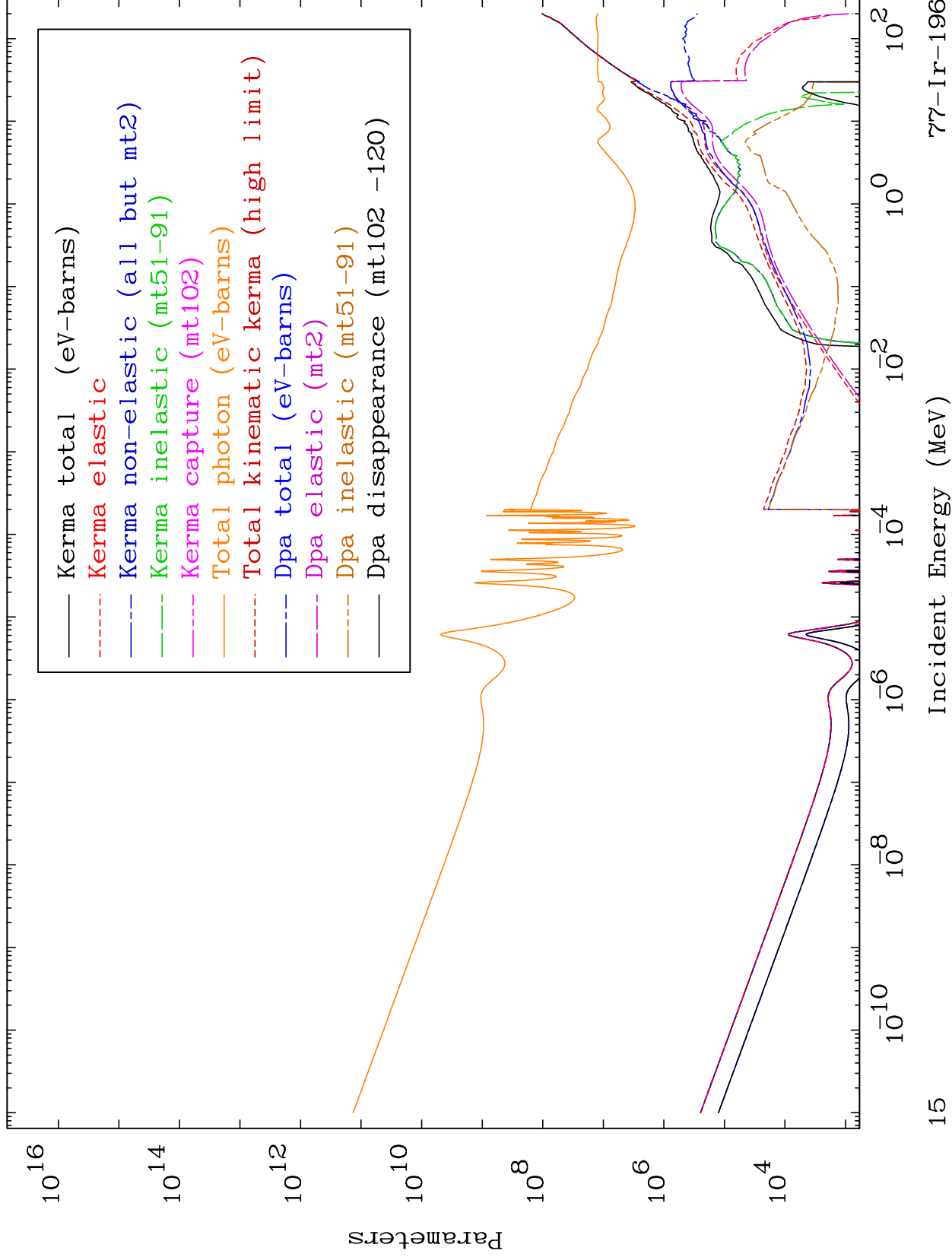
77-Ir-196

MAT 7741

(n,  $\alpha$ ) Levels  
293 Kelvin Cross Sections

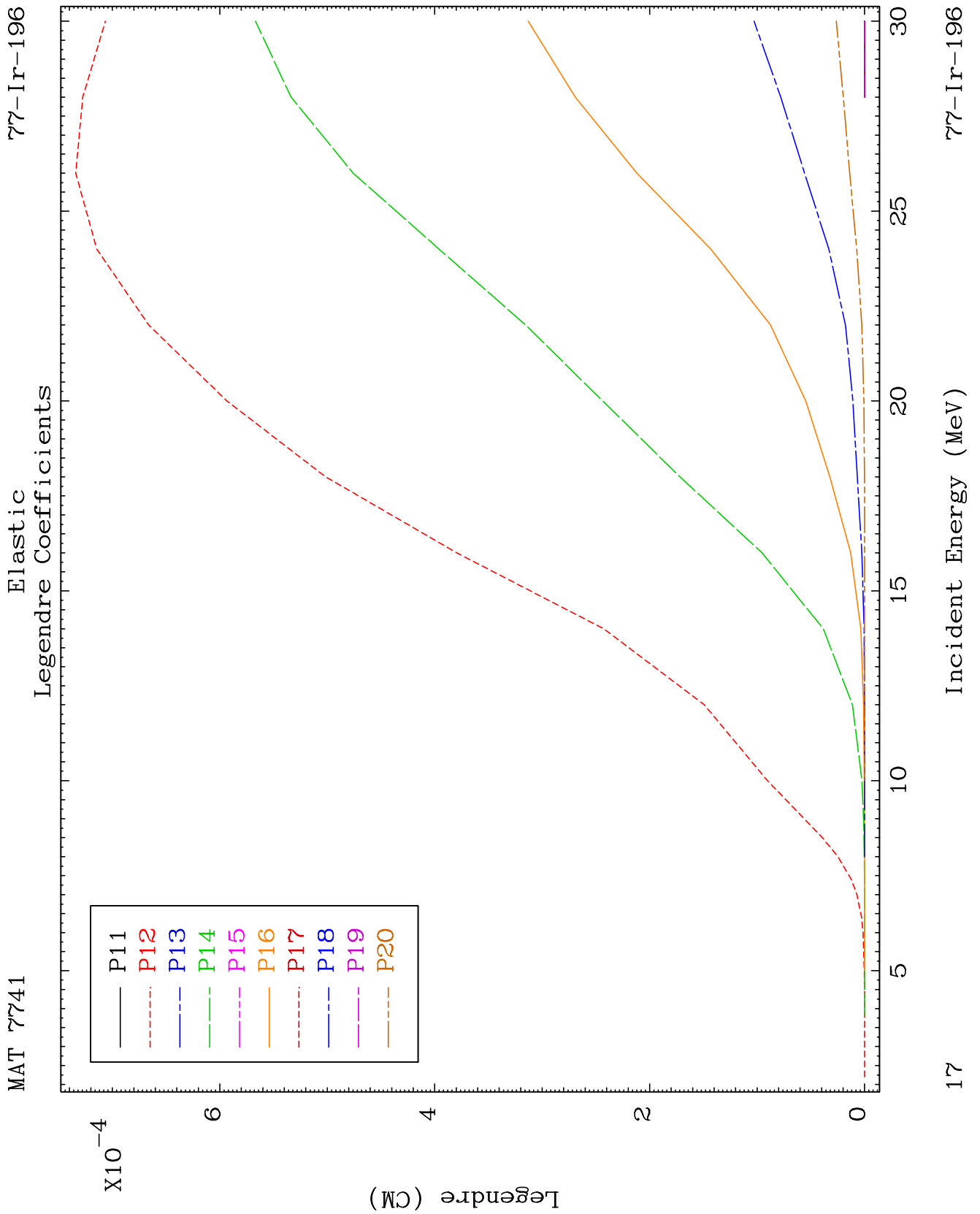
77-Ir-196







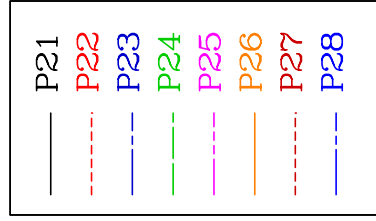




MAT 7741

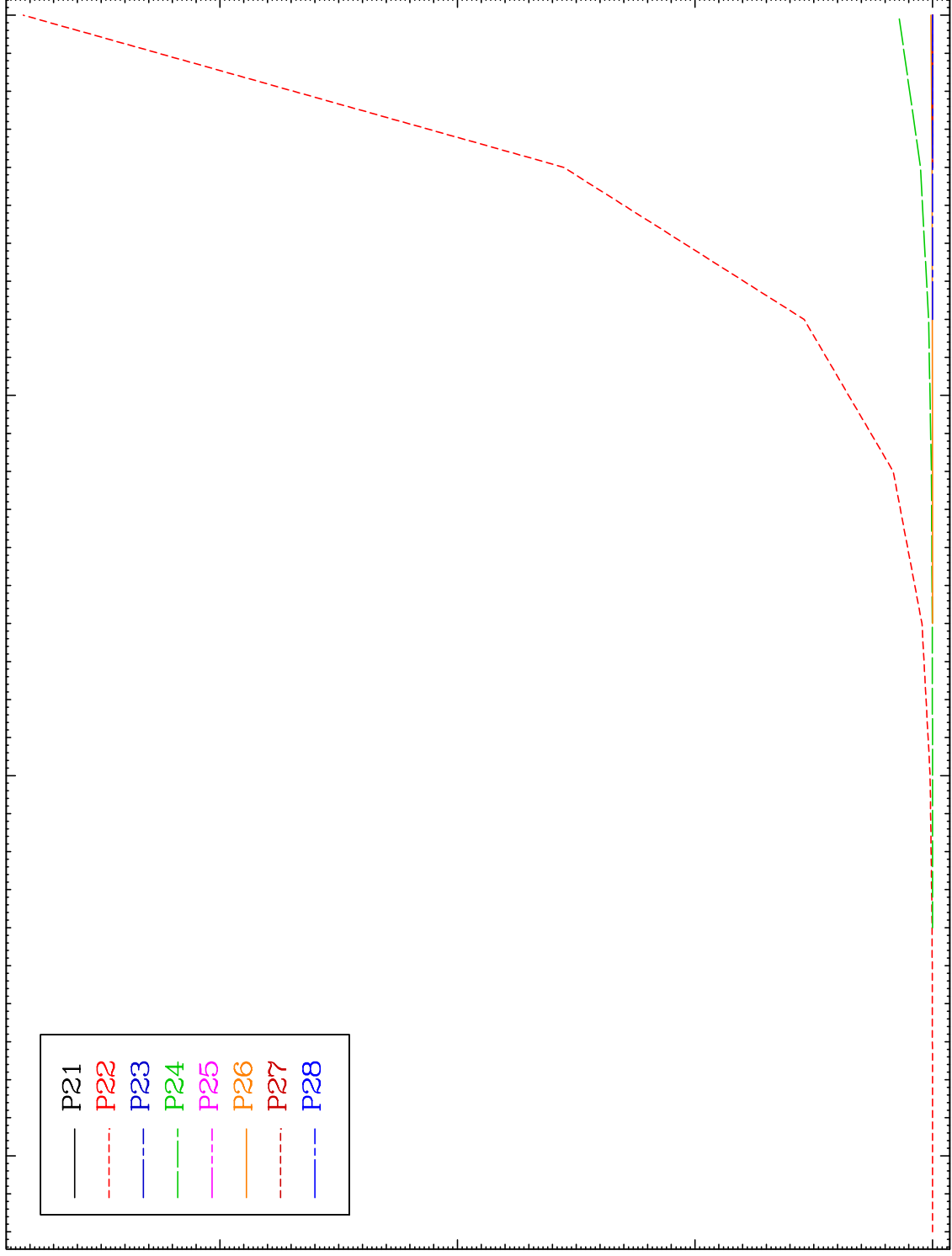
Elastic  
Legendre Coefficients

77-Ir-196



$\times 10^{-6}$

Legendre (CM)



18

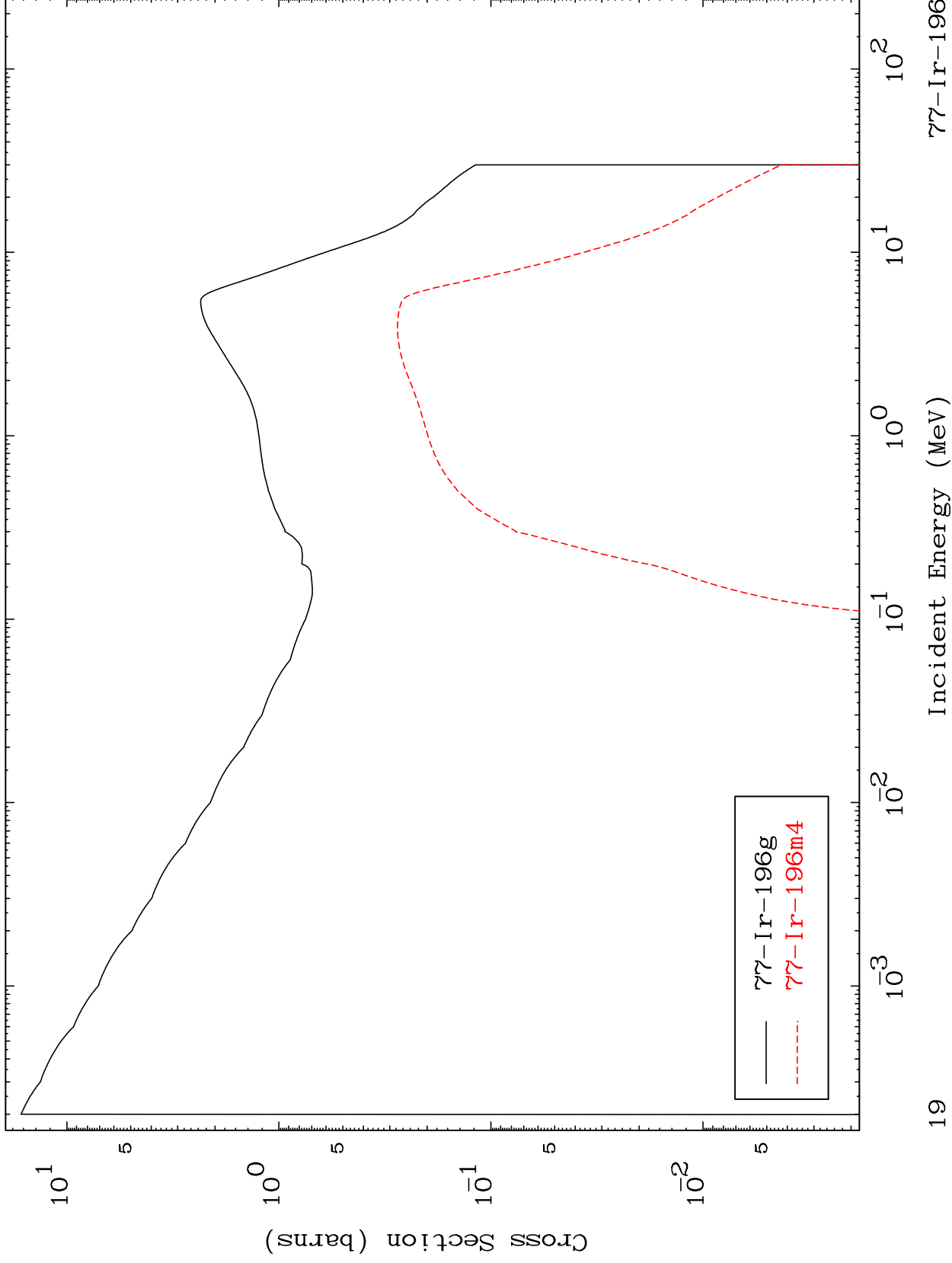
Incident Energy (MeV)

77-Ir-196

MAT 7741

Inelastic  
Radionuclide Production Cross Section

77-Ir-196

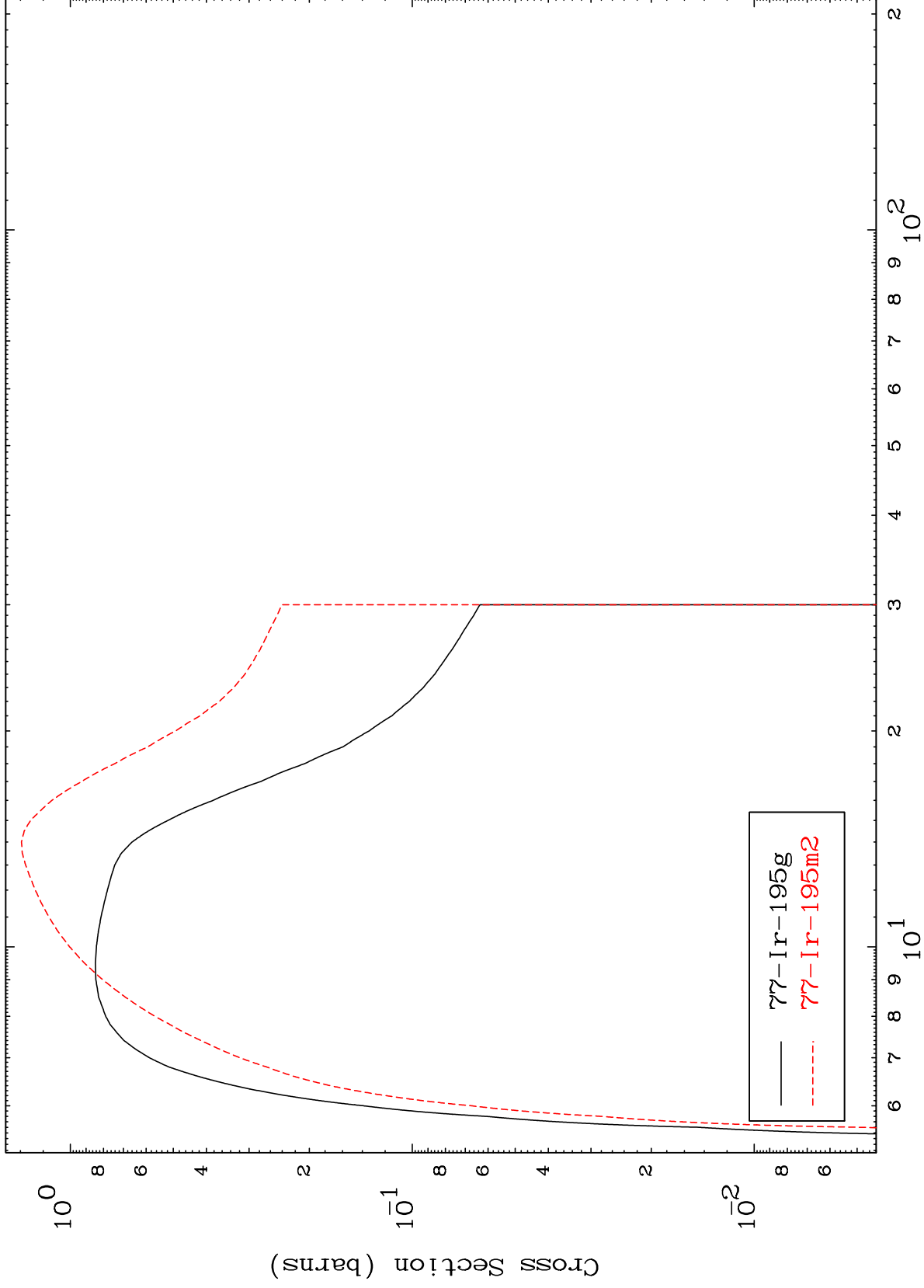


MAT 7741

(n,2n)

77-Ir-196

Radionuclide Production Cross Section



20

Incident Energy (MeV)

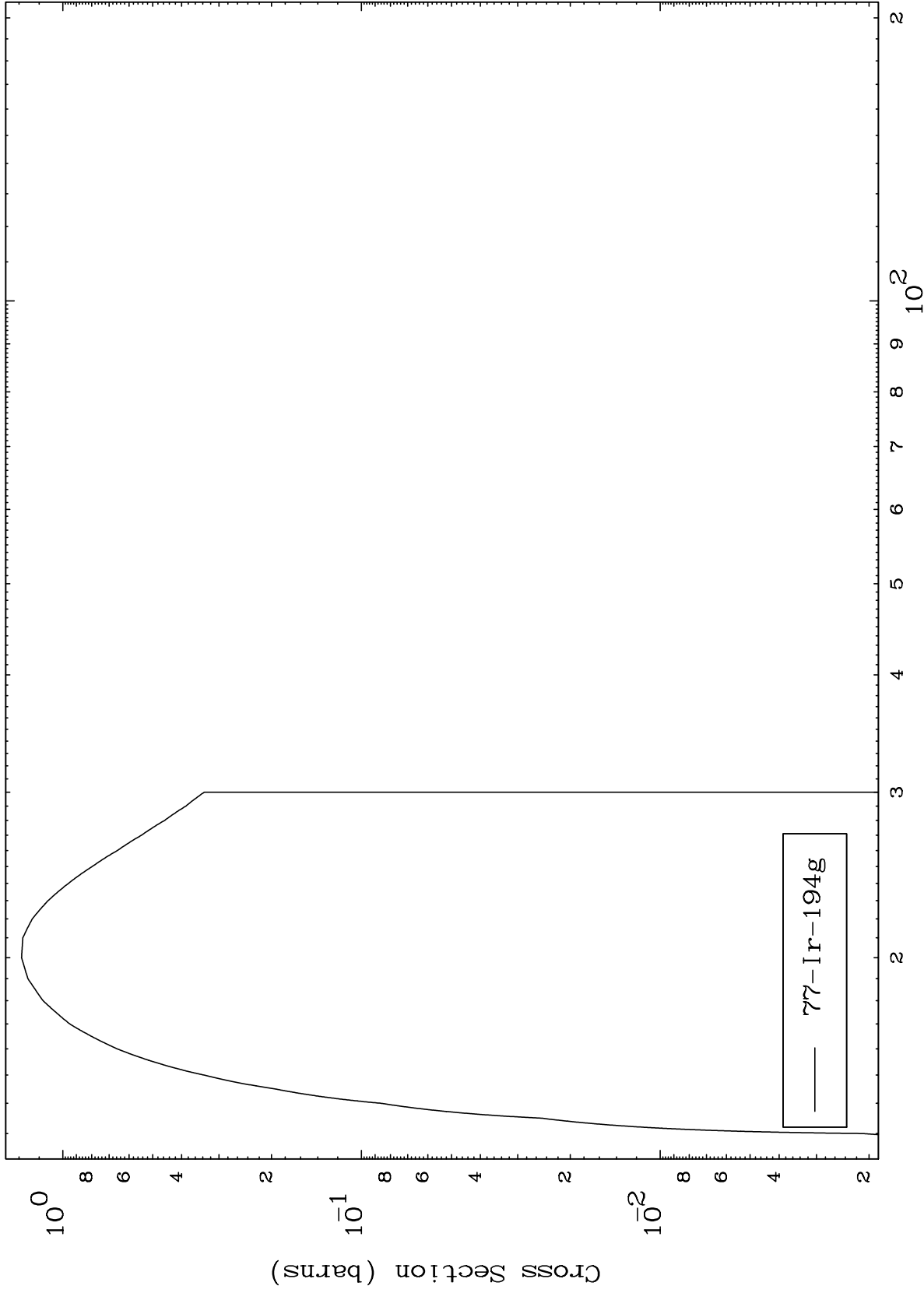
77-Ir-196

MAT 7741

(n,3n)

<sup>77</sup>Ir-196

Radionuclide Production Cross Section



21

Incident Energy (MeV)

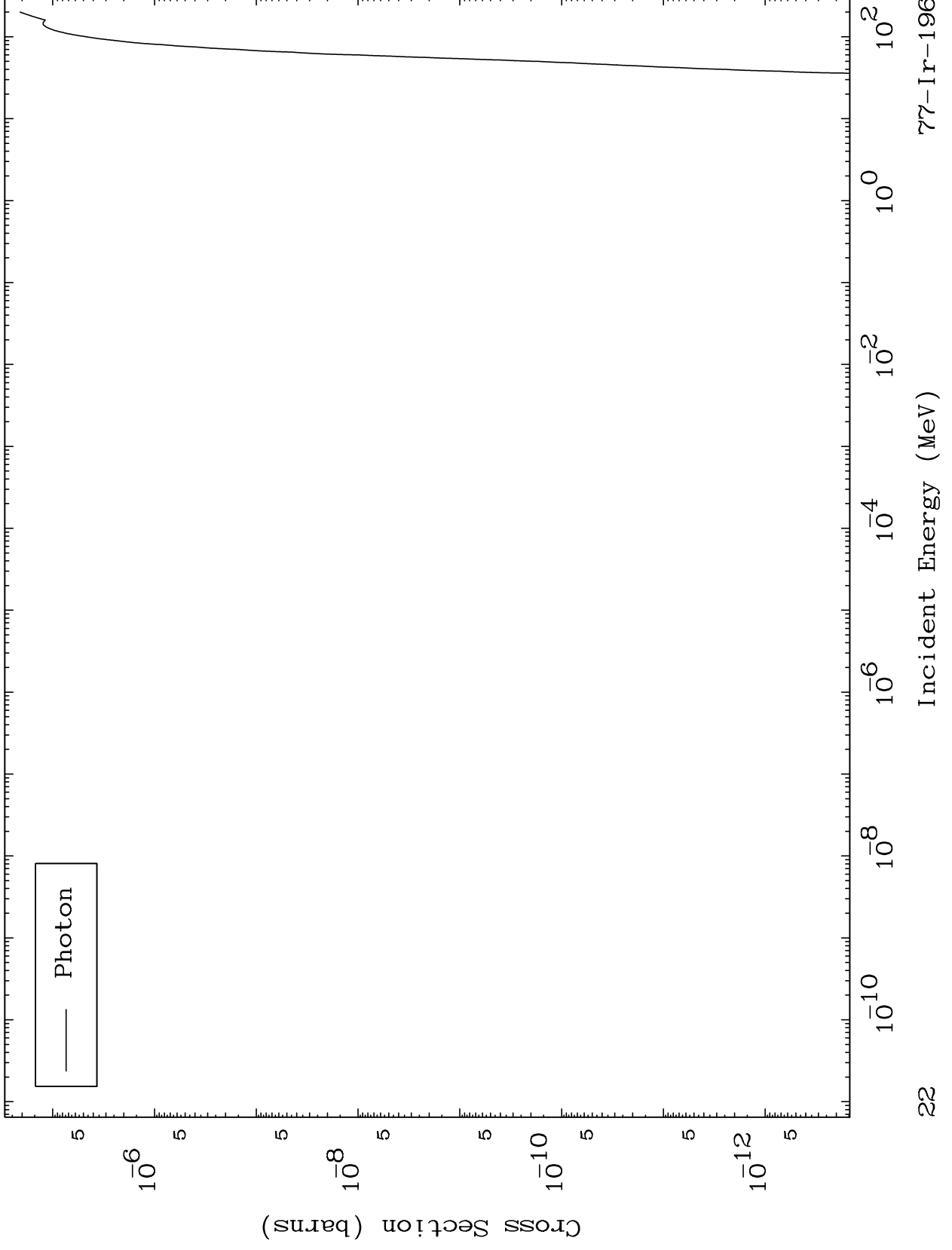
<sup>77</sup>Ir-196

MAT 7741

Fission

<sup>77</sup>Ir-196

Radionuclide Production Cross Section

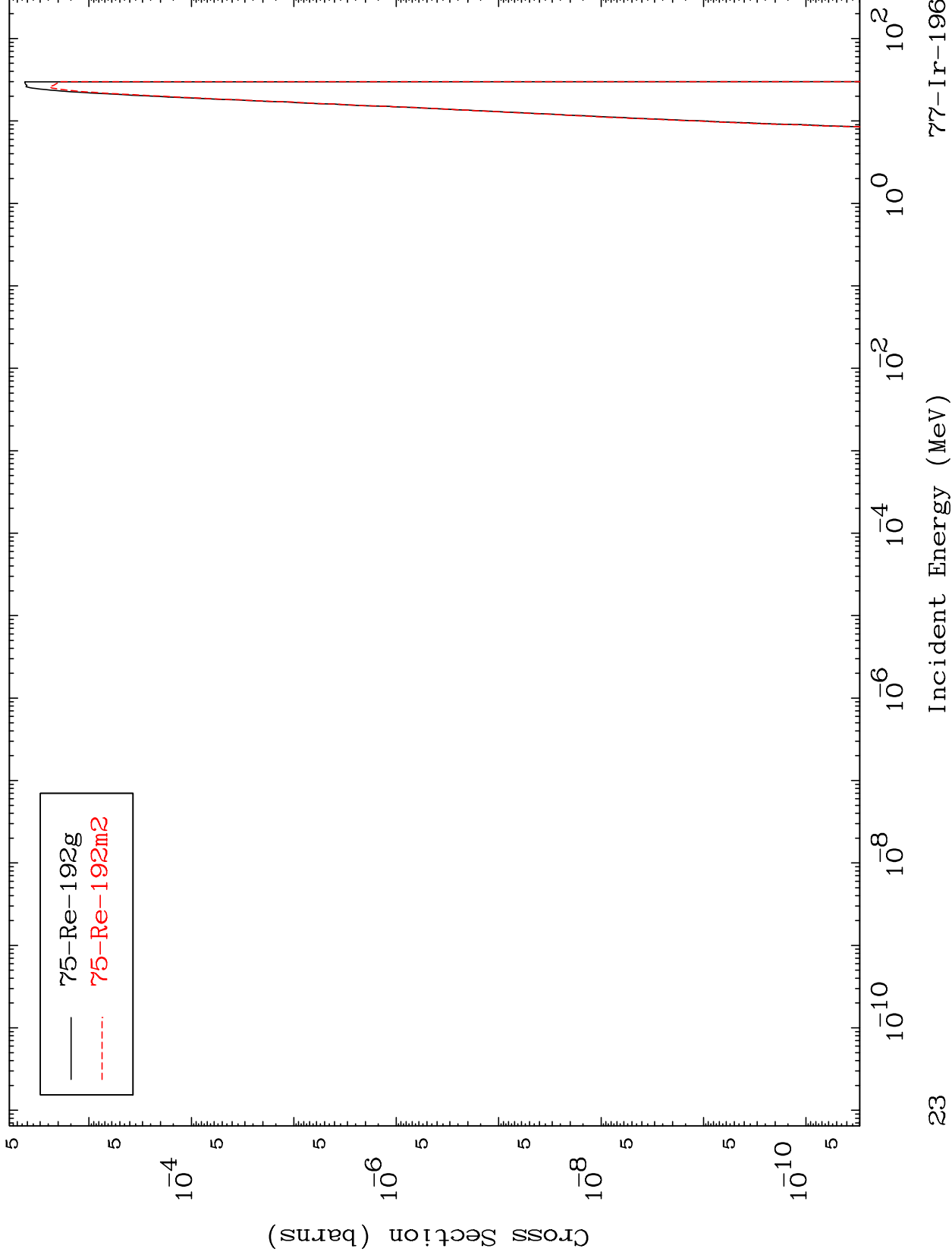


MAT 7741

$(n, n') \alpha$

$^{77}\text{Ir-196}$

Radionuclide Production Cross Section



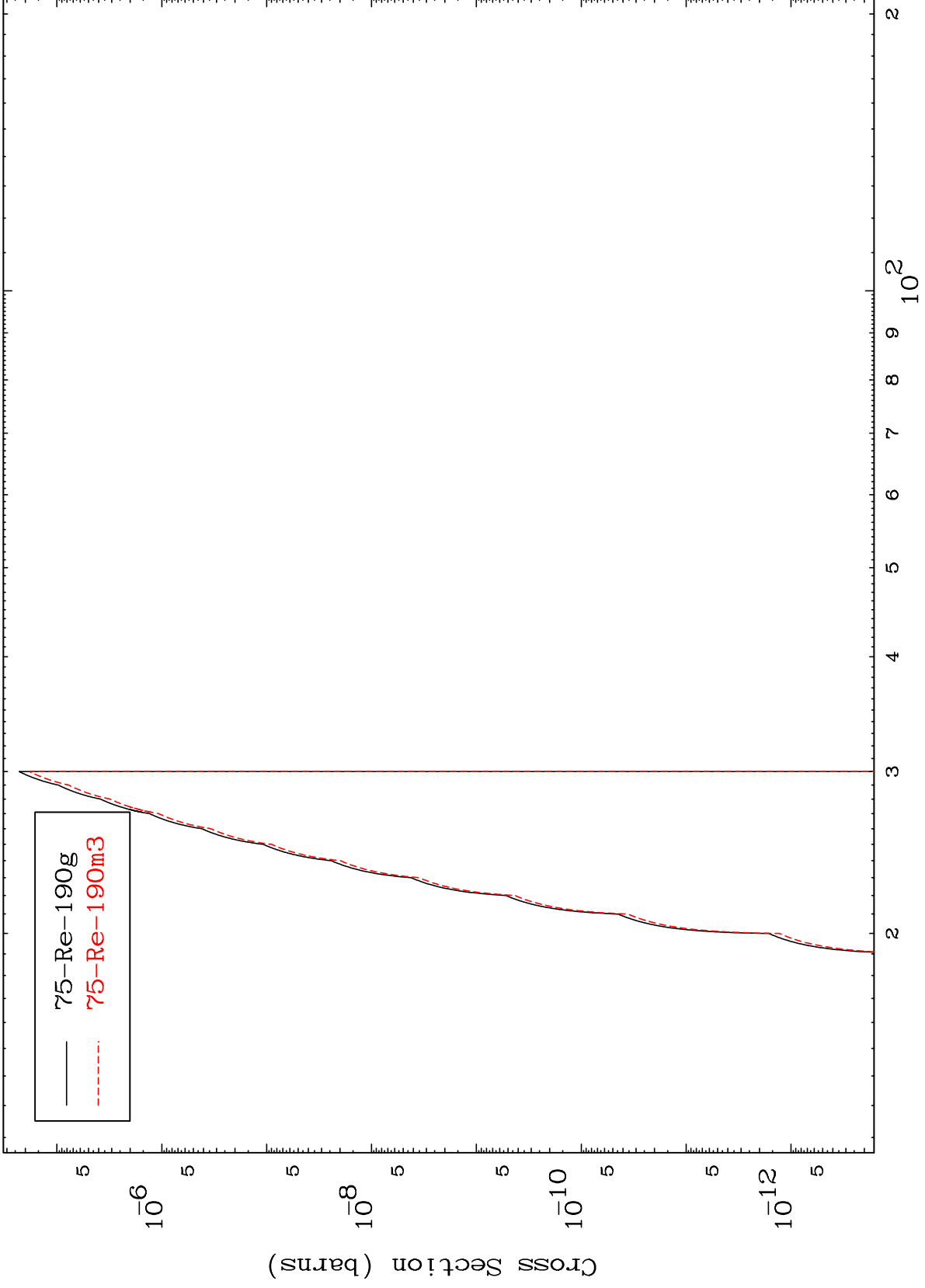


MAT 7741

(n,3n)  $\alpha$

77-Ir-196

Radionuclide Production Cross Section



24

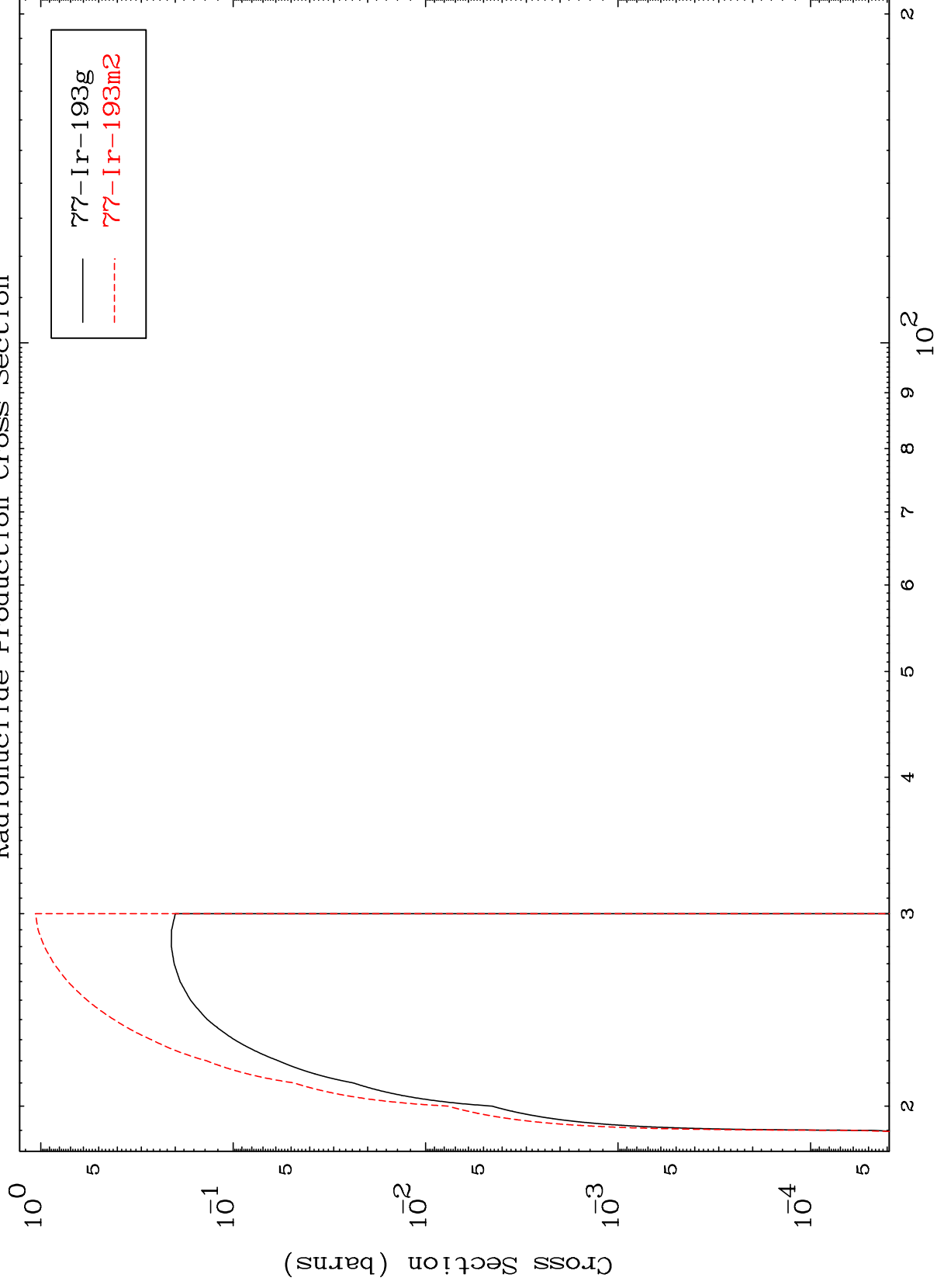
Incident Energy (MeV)

77-Ir-196

MAT 7741

77-Ir-196

(n,4n)  
Radionuclide Production Cross Section



25

Incident Energy (MeV)

77-Ir-196

MAT 7741

77-Ir-196

$(n, \gamma)$   
Radionuclide Production Cross Section

