

Program EVALPLOT
(Version 2015-2)

by

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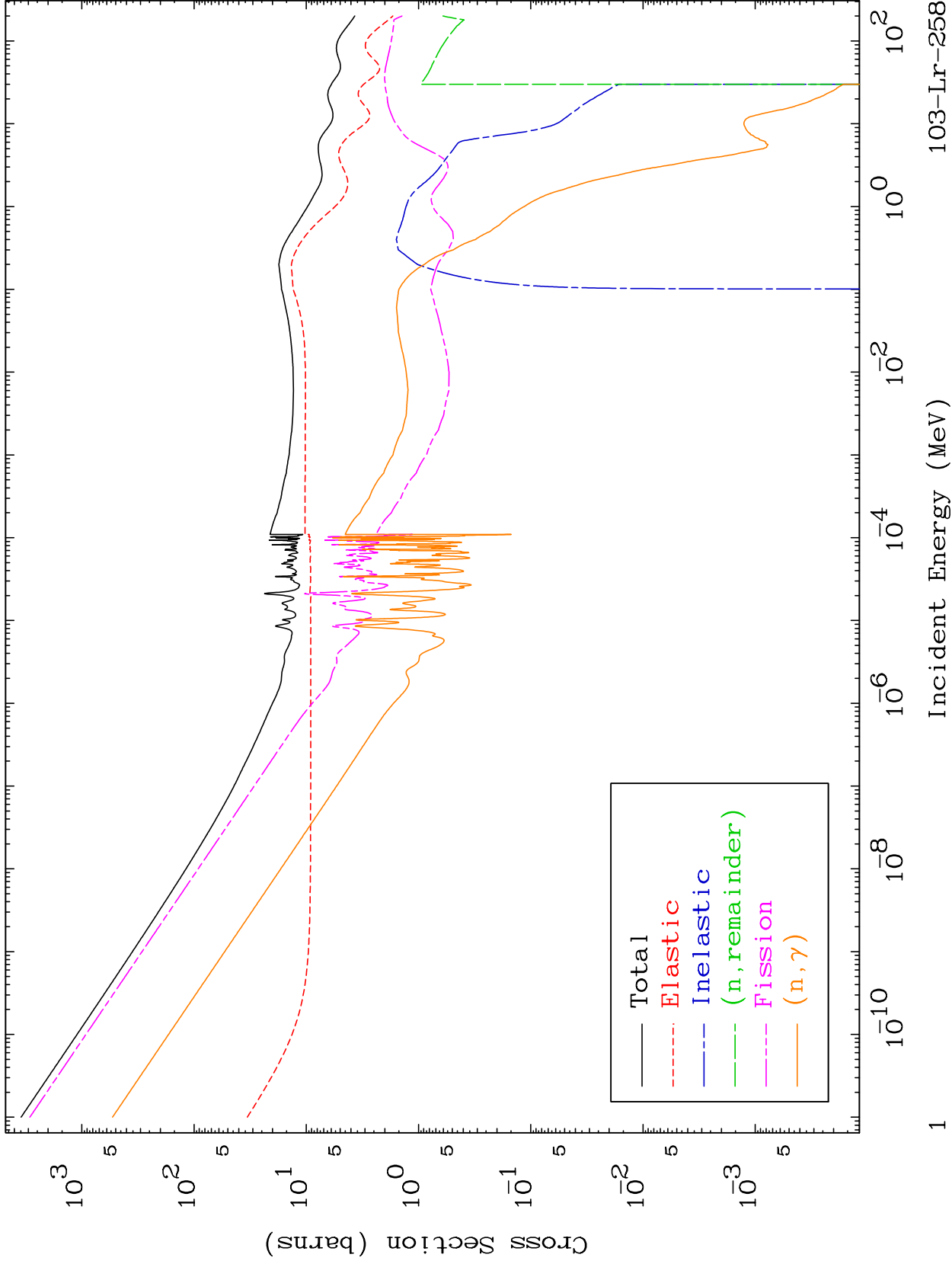
Press Mouse Button to Start

MAT 9987

Major

293 Kelvin Cross Sections

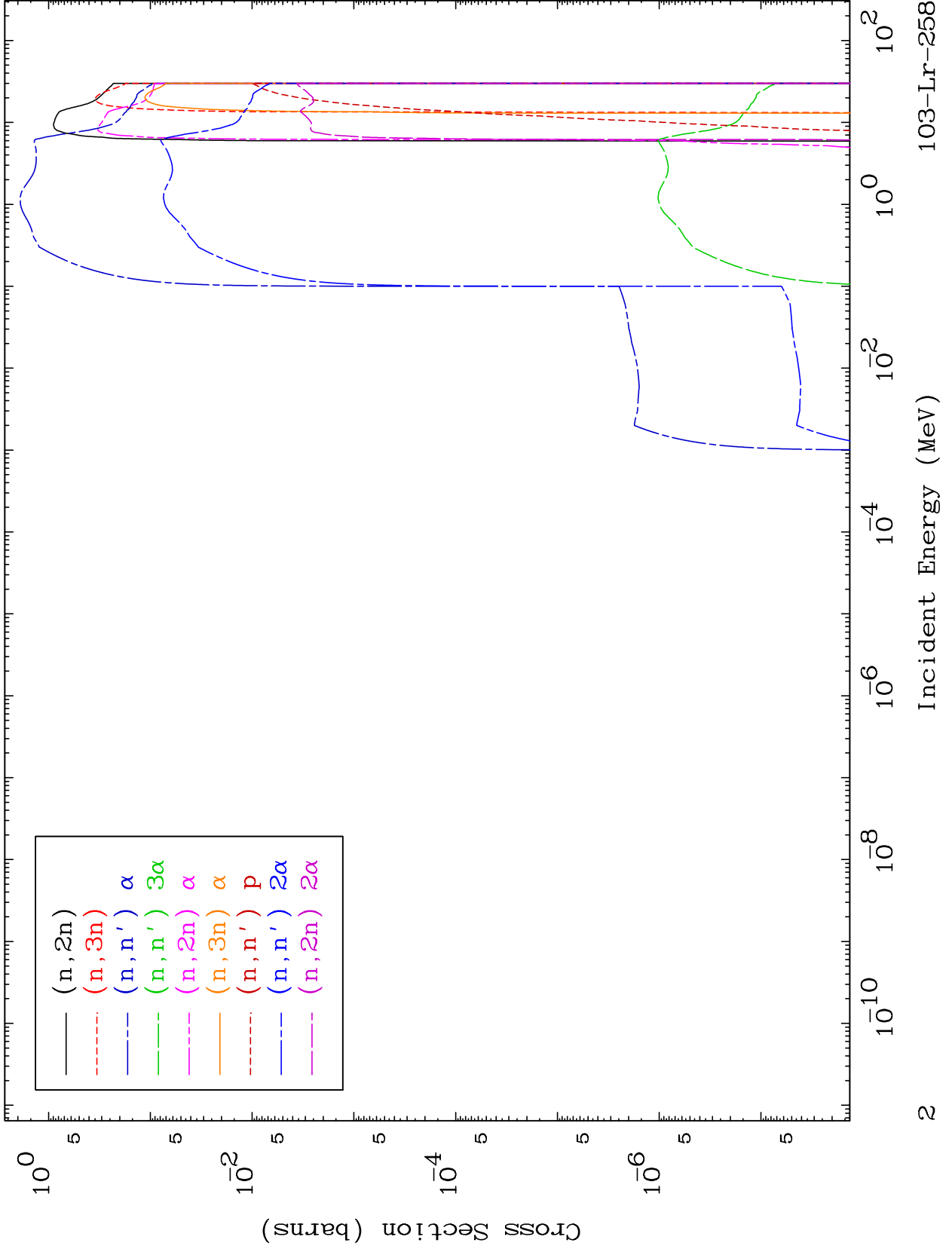
103-Lr-258



MAT 9987

Neutron Production
293 Kelvin Cross Sections

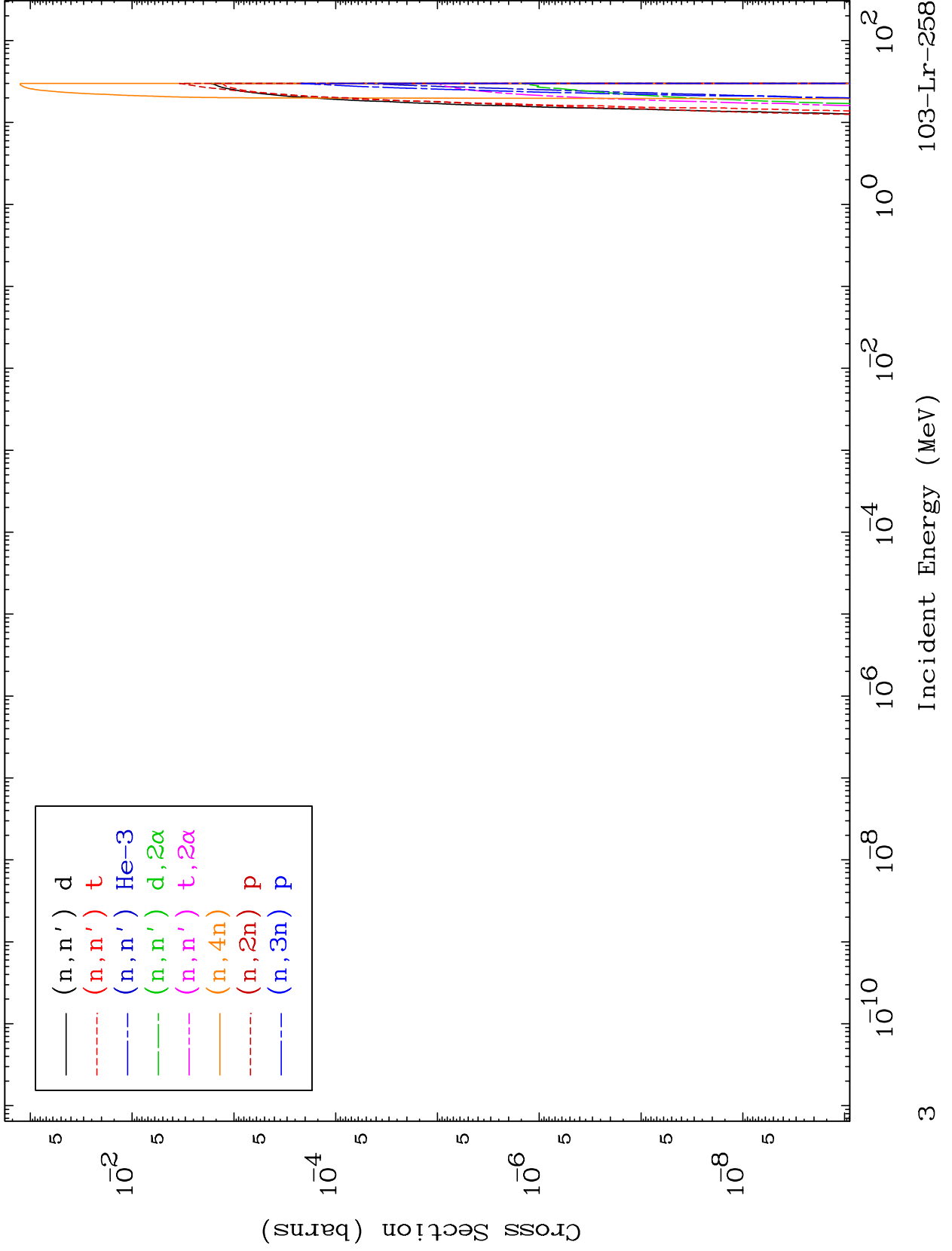
103-Lr-258



MAT 9987

Neutron Production
293 Kelvin Cross Sections

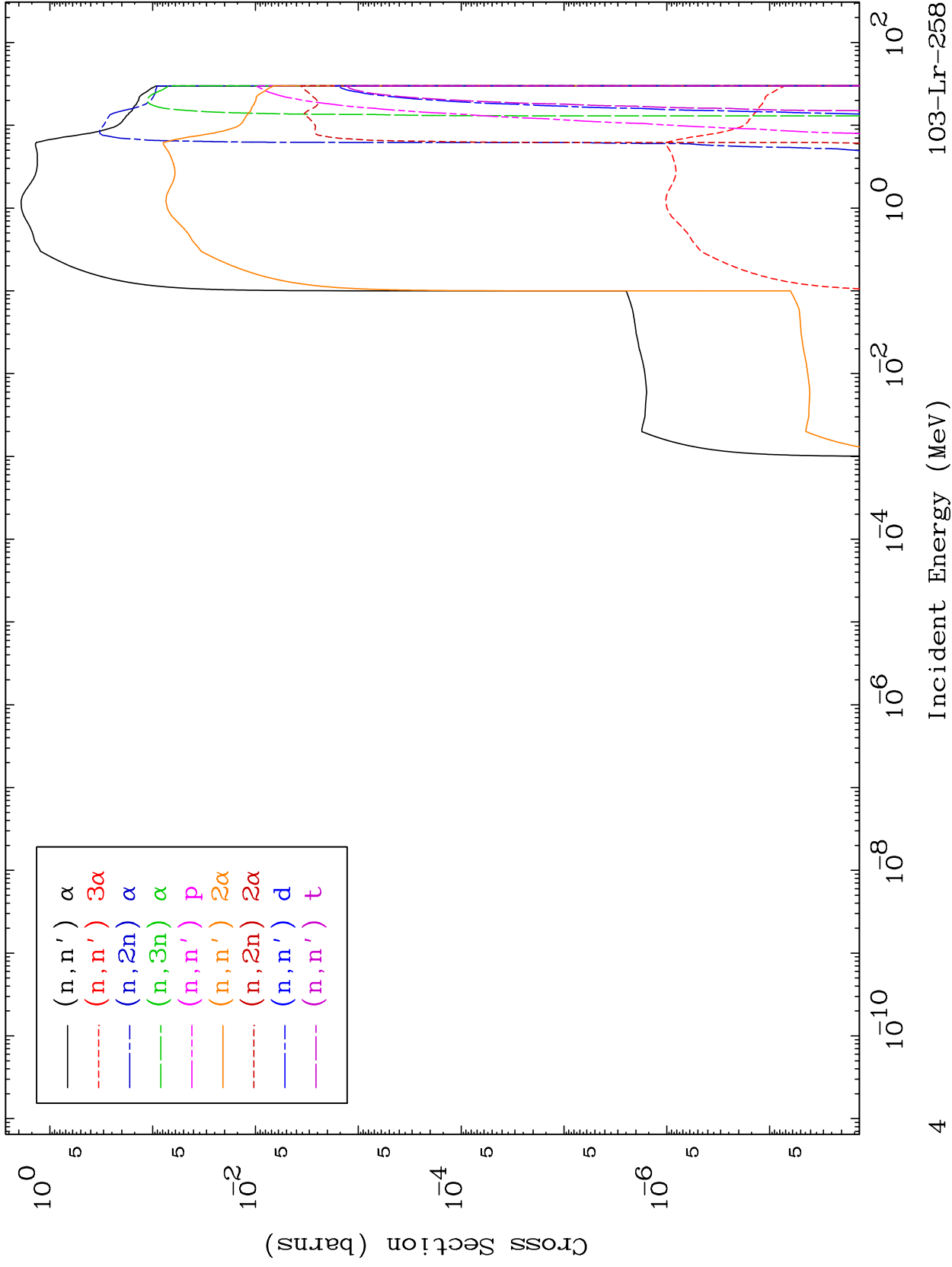
103-Lr-258



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Charged Particle
293 Kelvin Cross Sections

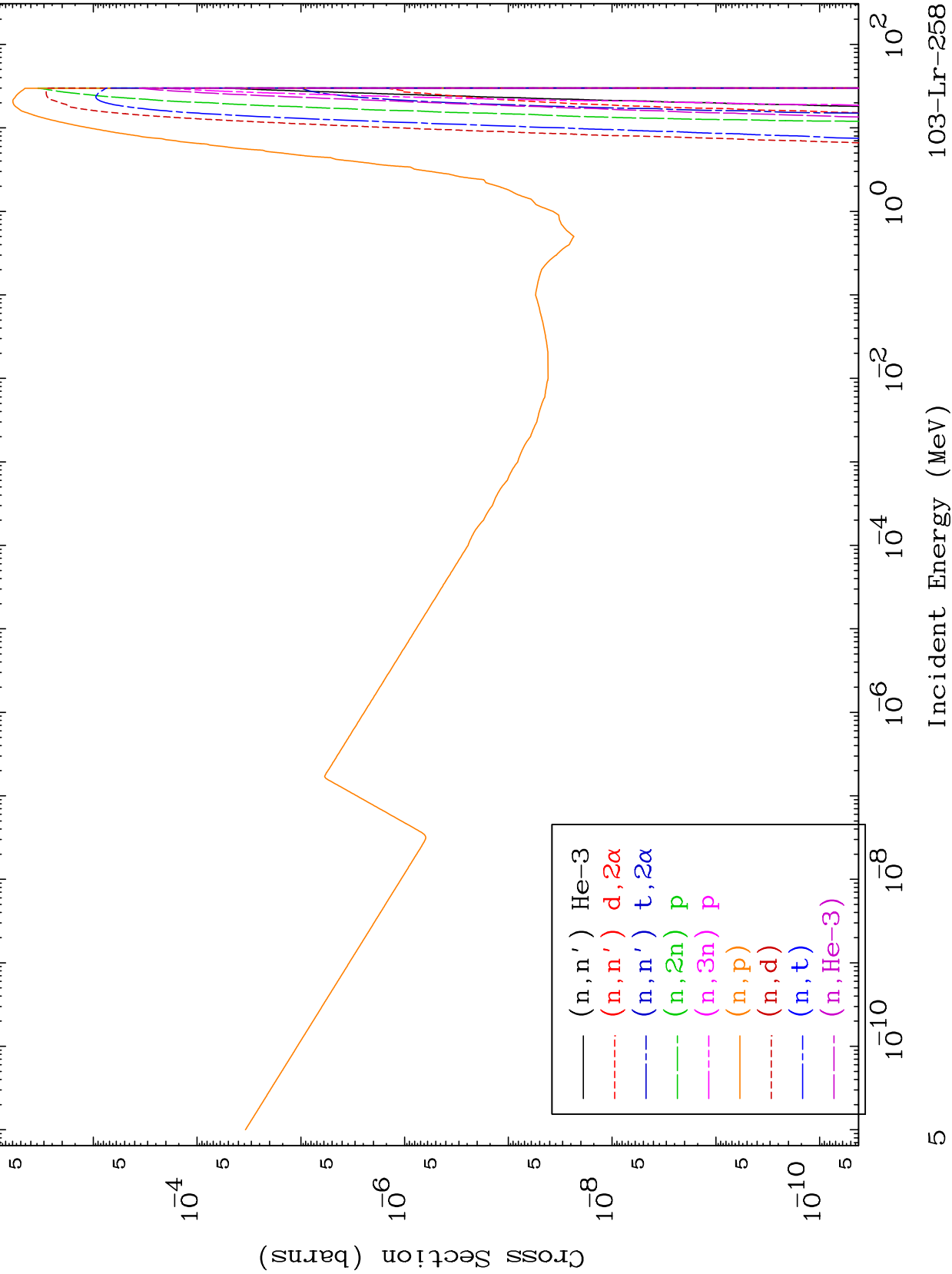
103-Lr-258



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Charged Particle
293 Kelvin Cross Sections

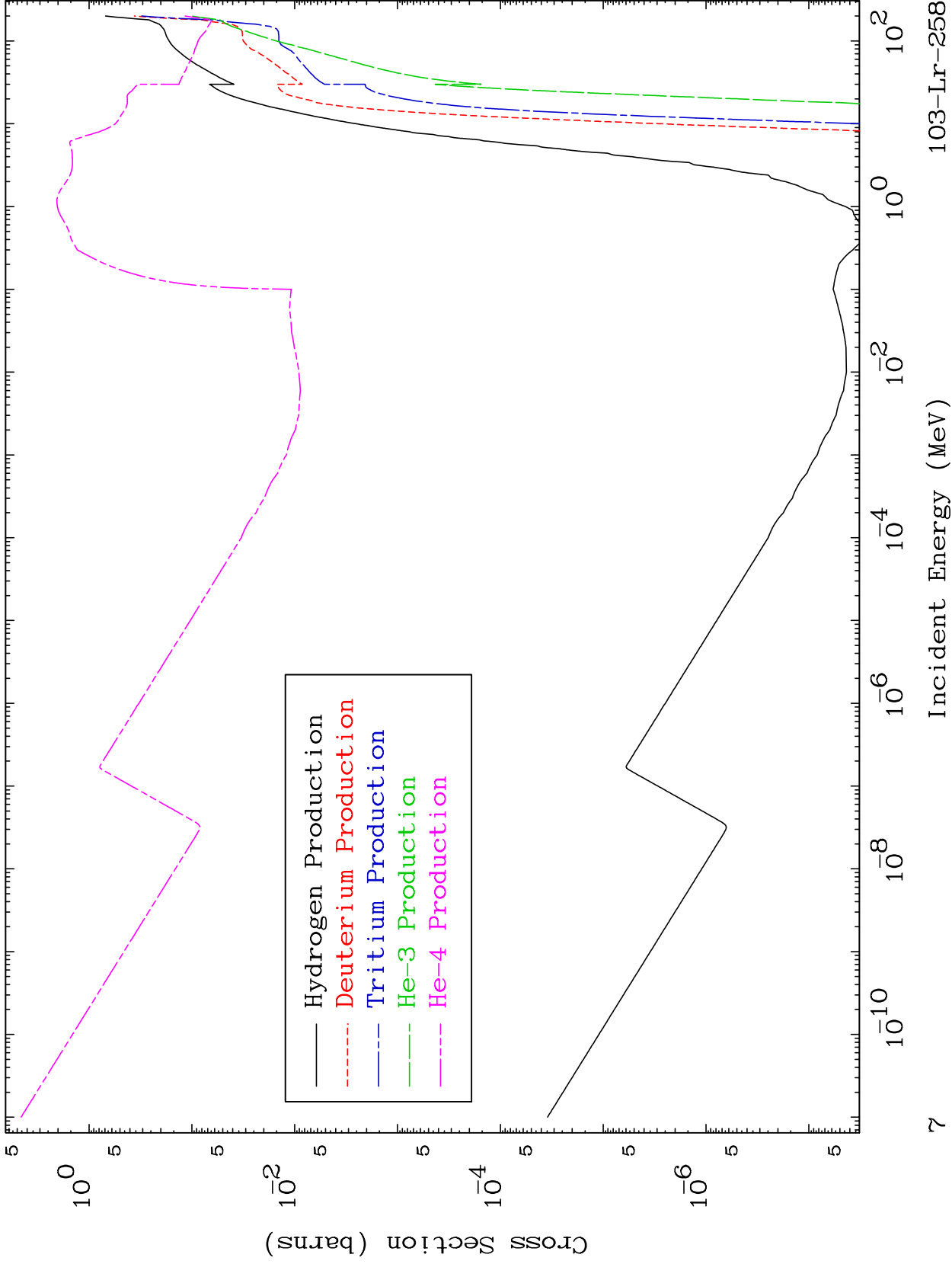
103-Lr-258



MAT 9987

Particle Production
293 Kelvin Cross Sections

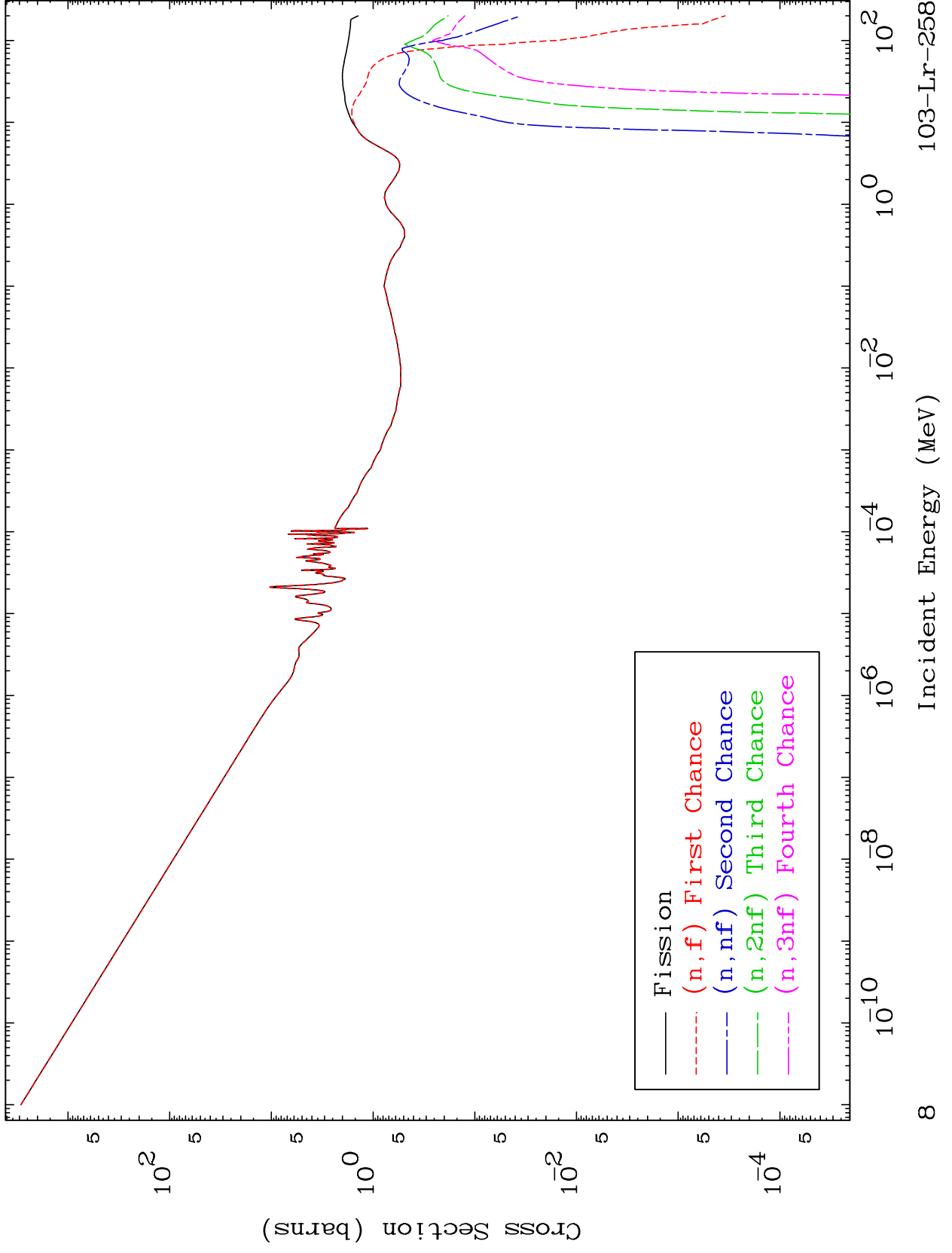
103-Lr-258



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Fission
293 Kelvin Cross Sections

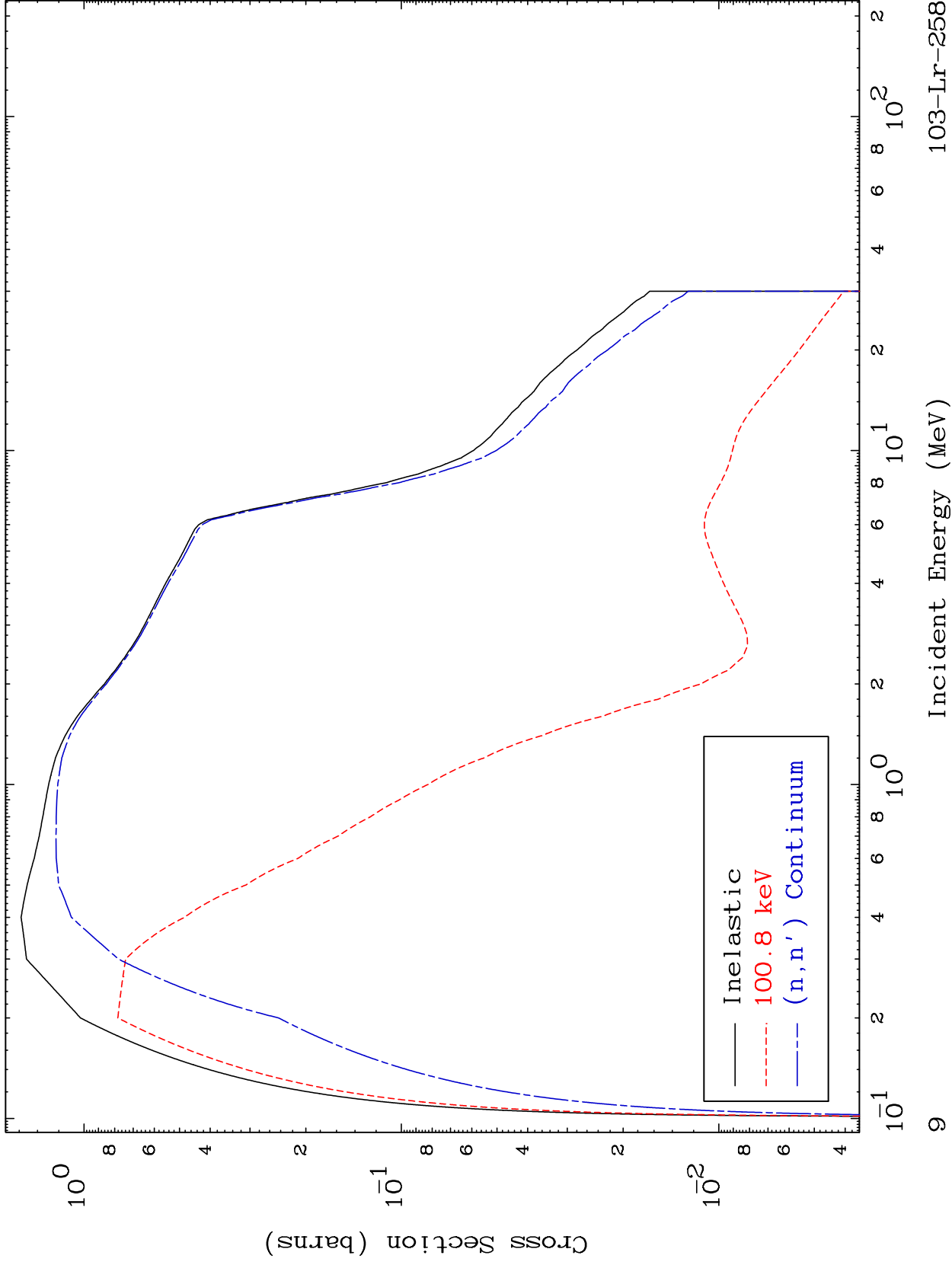
103-Lr-258



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(n,n') Level
293 Kelvin Cross Sections

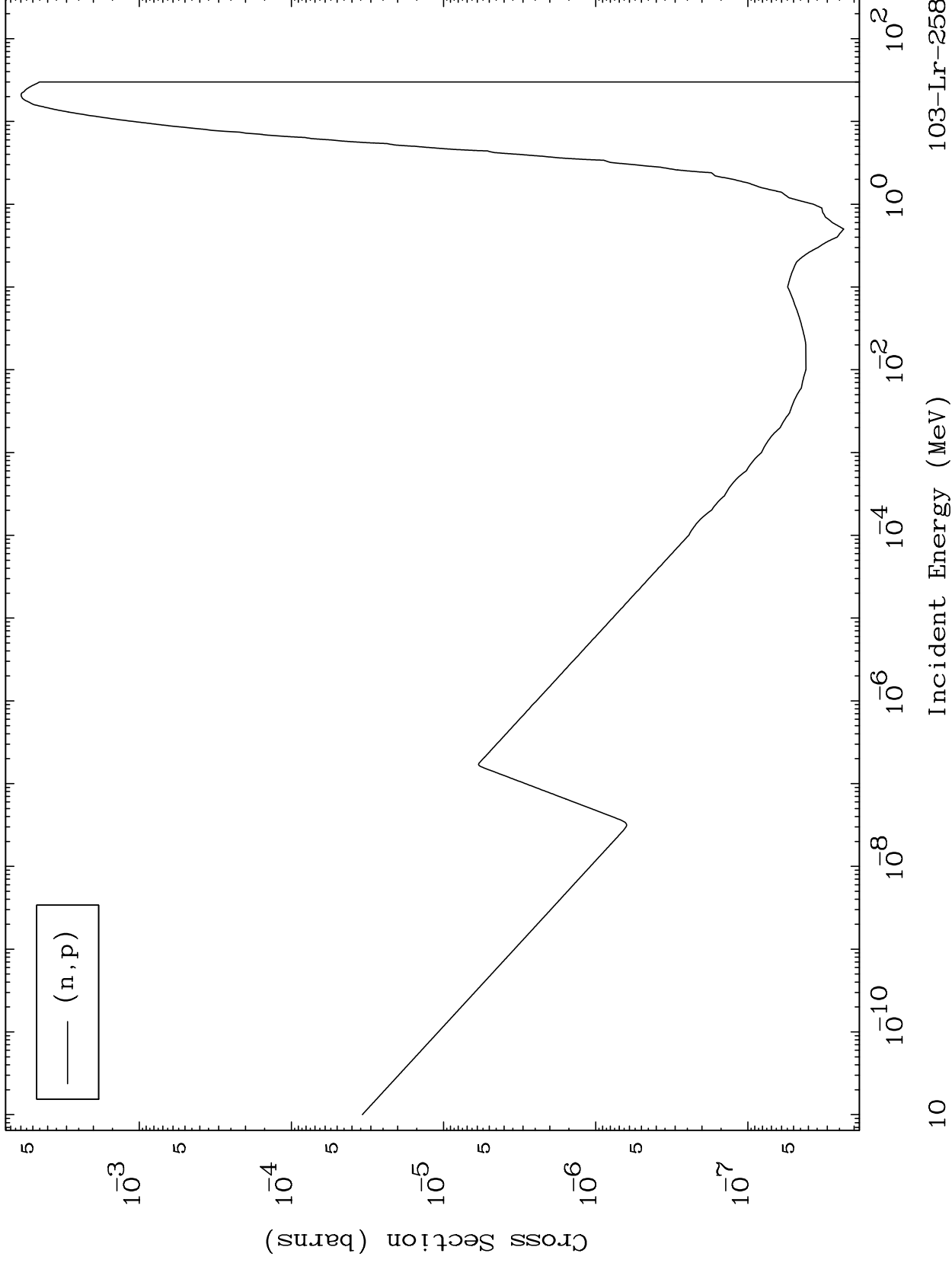
103-Lr-258



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(n,p) Levels
293 Kelvin Cross Sections

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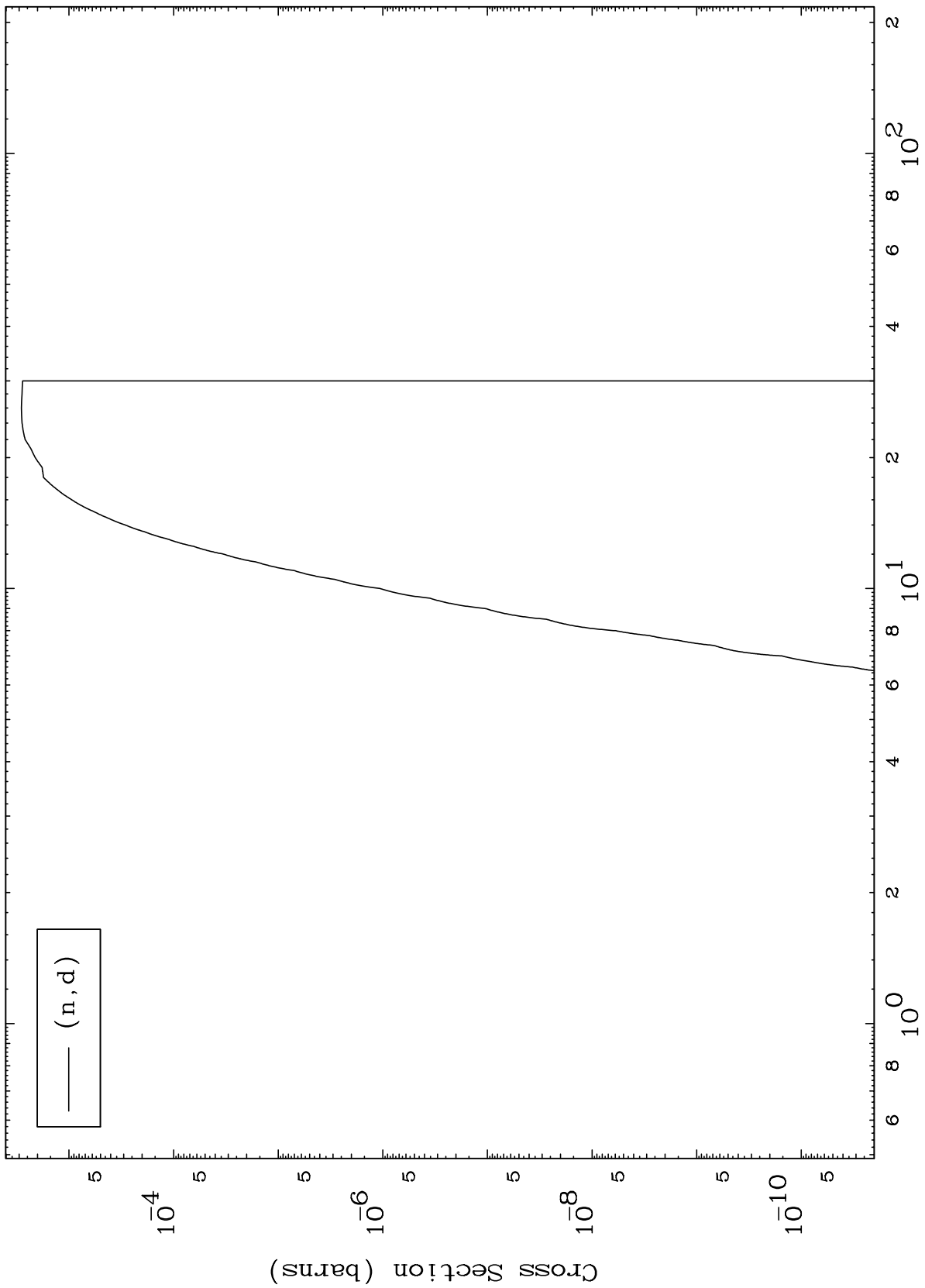
Incident Energy (MeV)

103-Lr-258

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(n,d) Levels
293 Kelvin Cross Sections

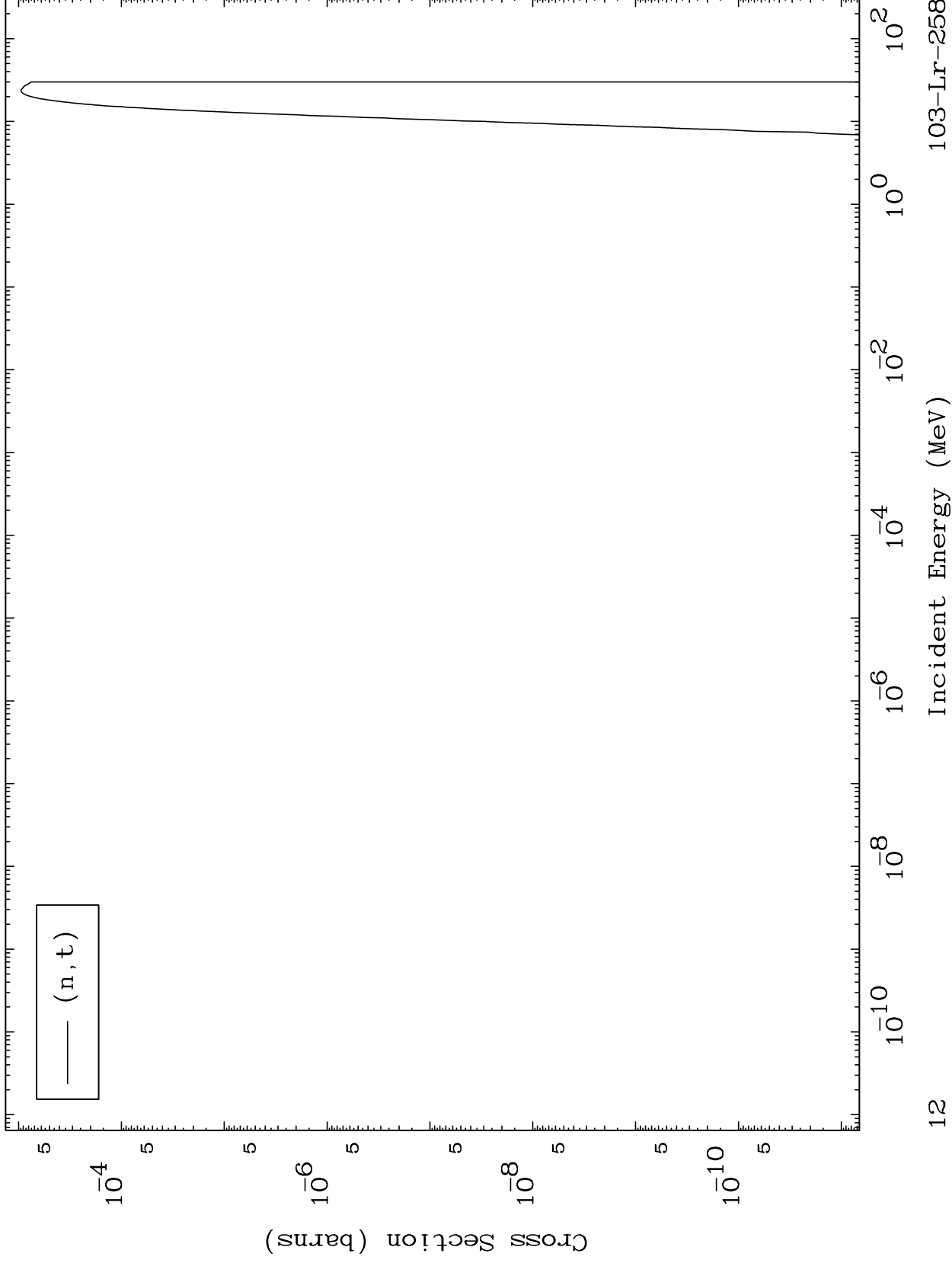
103-Lr-258



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(n,t) Levels
293 Kelvin Cross Sections

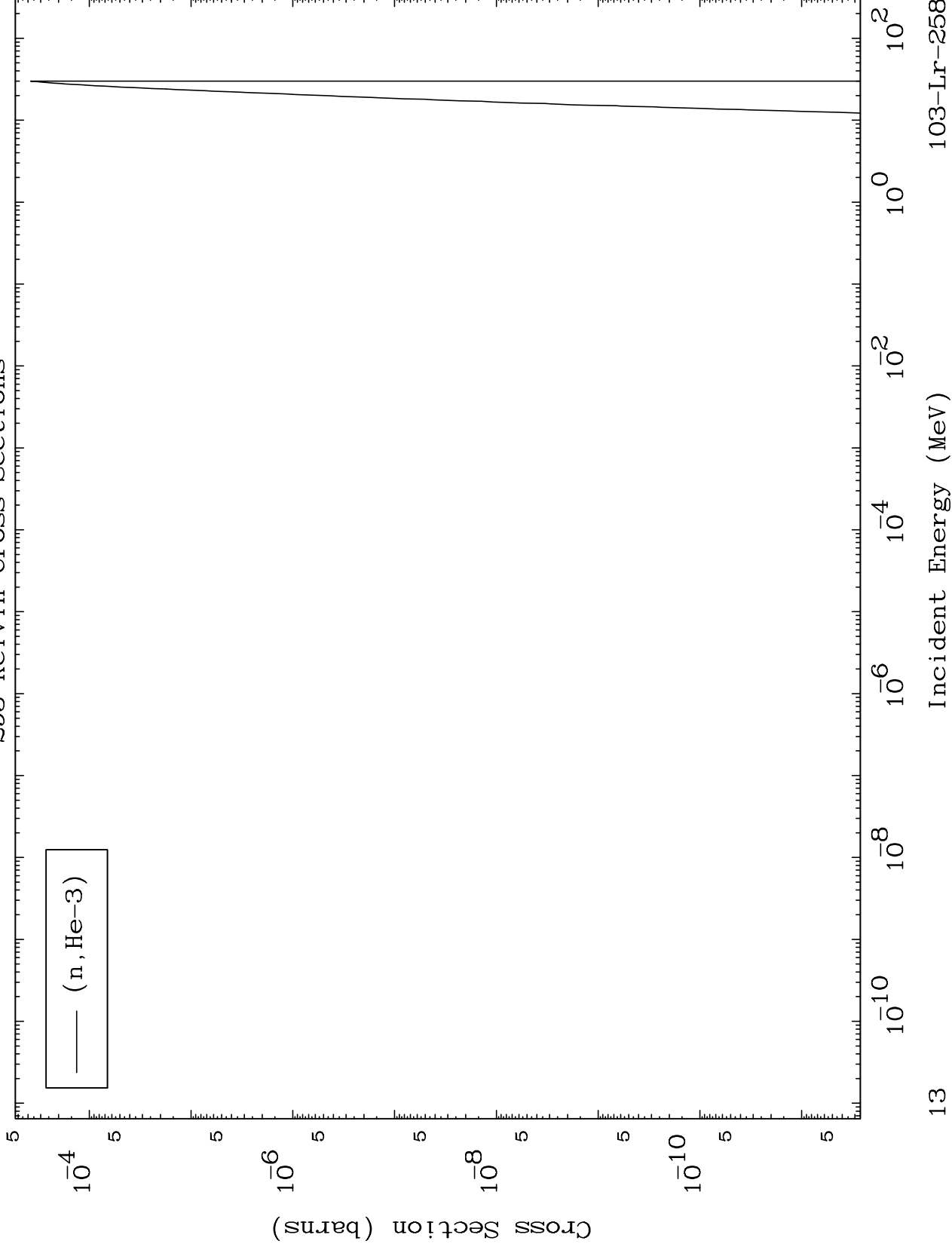
103-Lr-258



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(n,He3) Levels
293 Kelvin Cross Sections

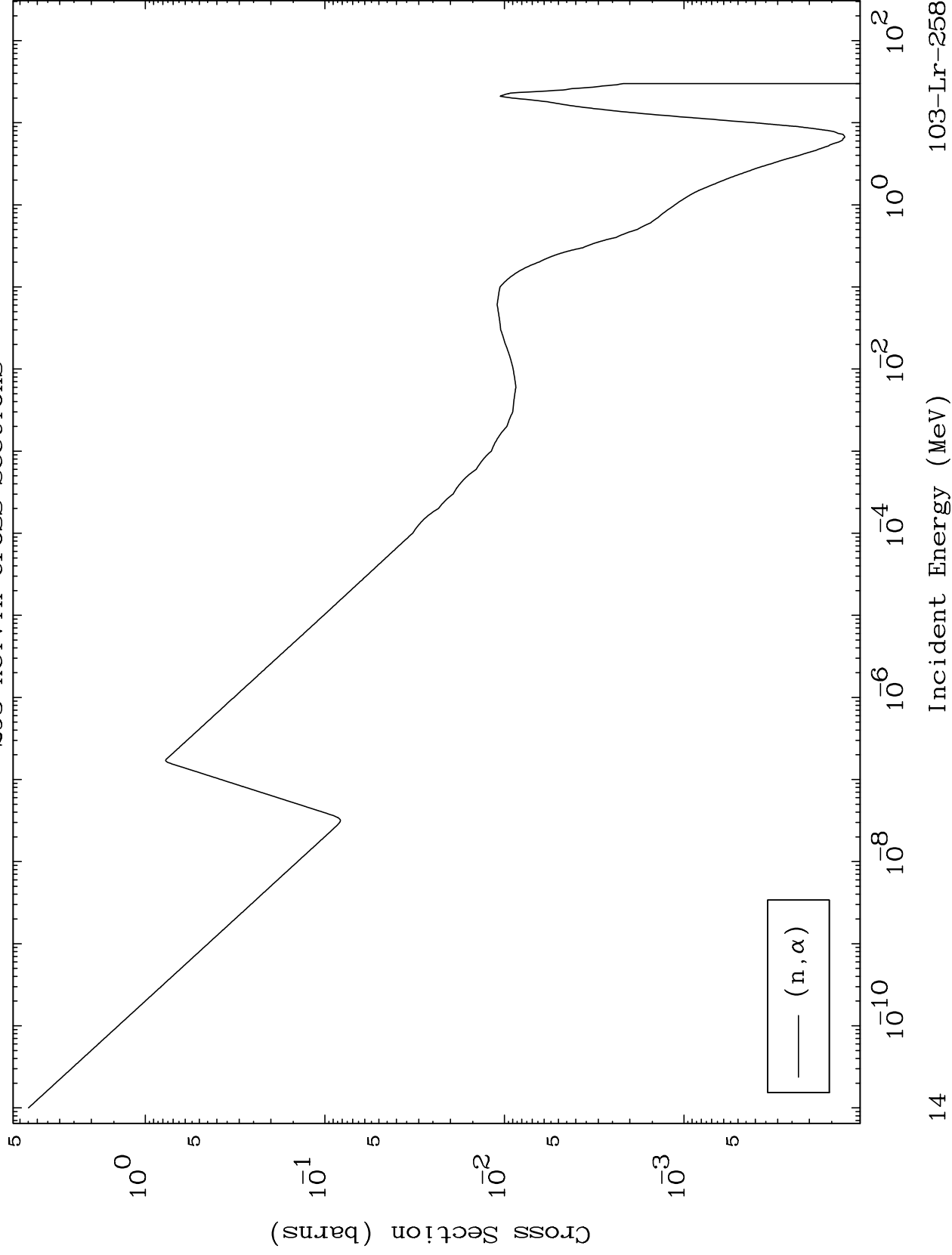
103-Lr-258

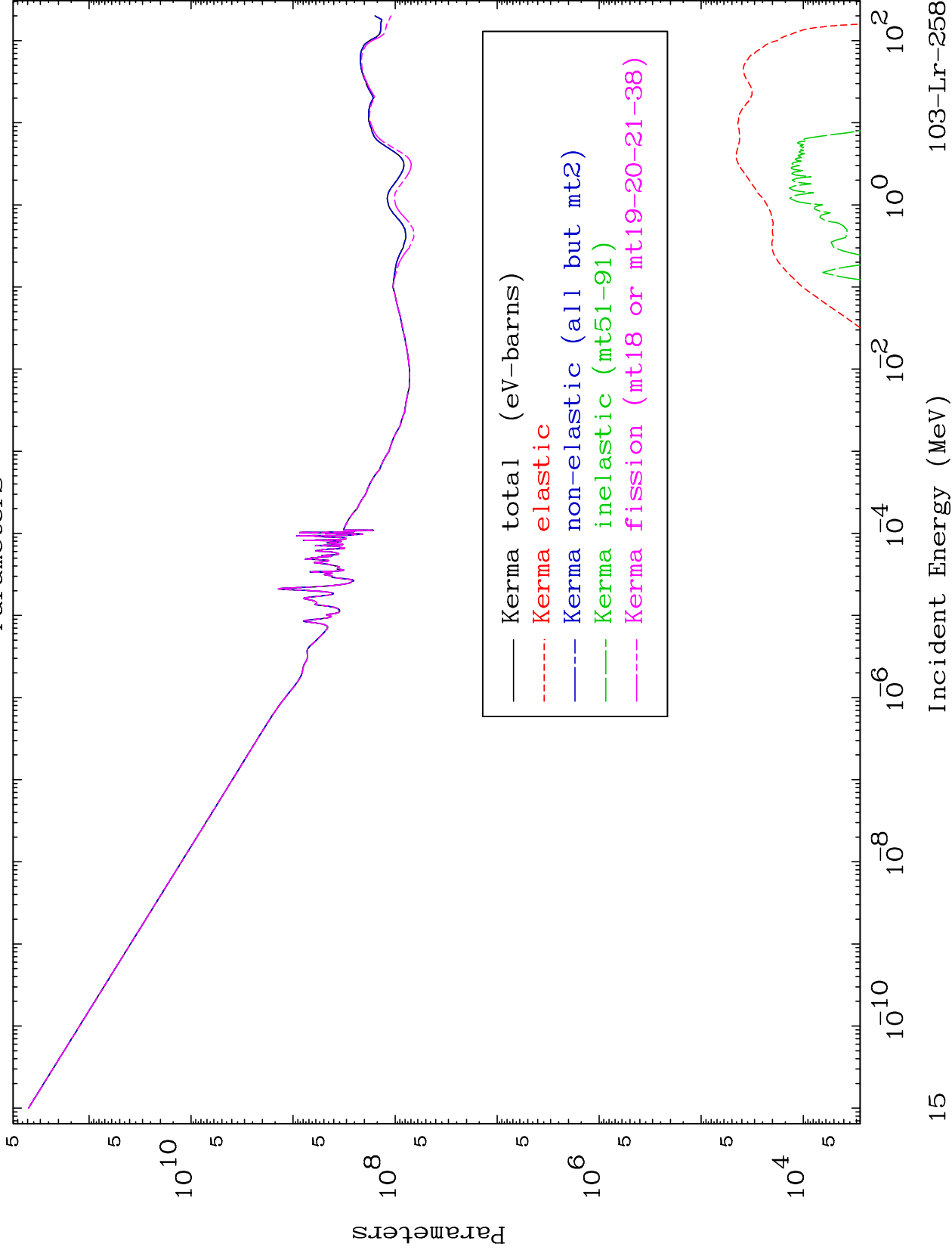


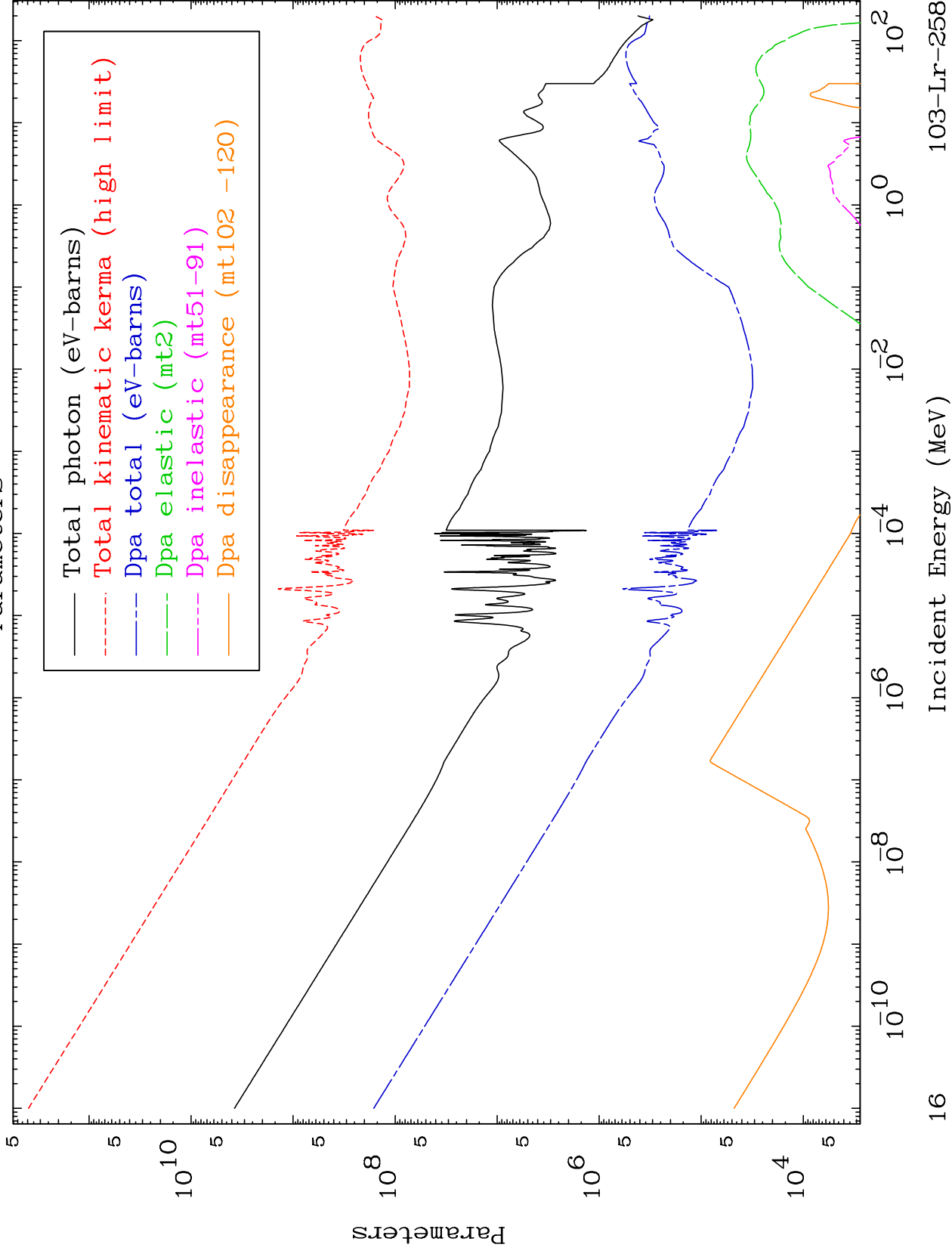
MAT 9987

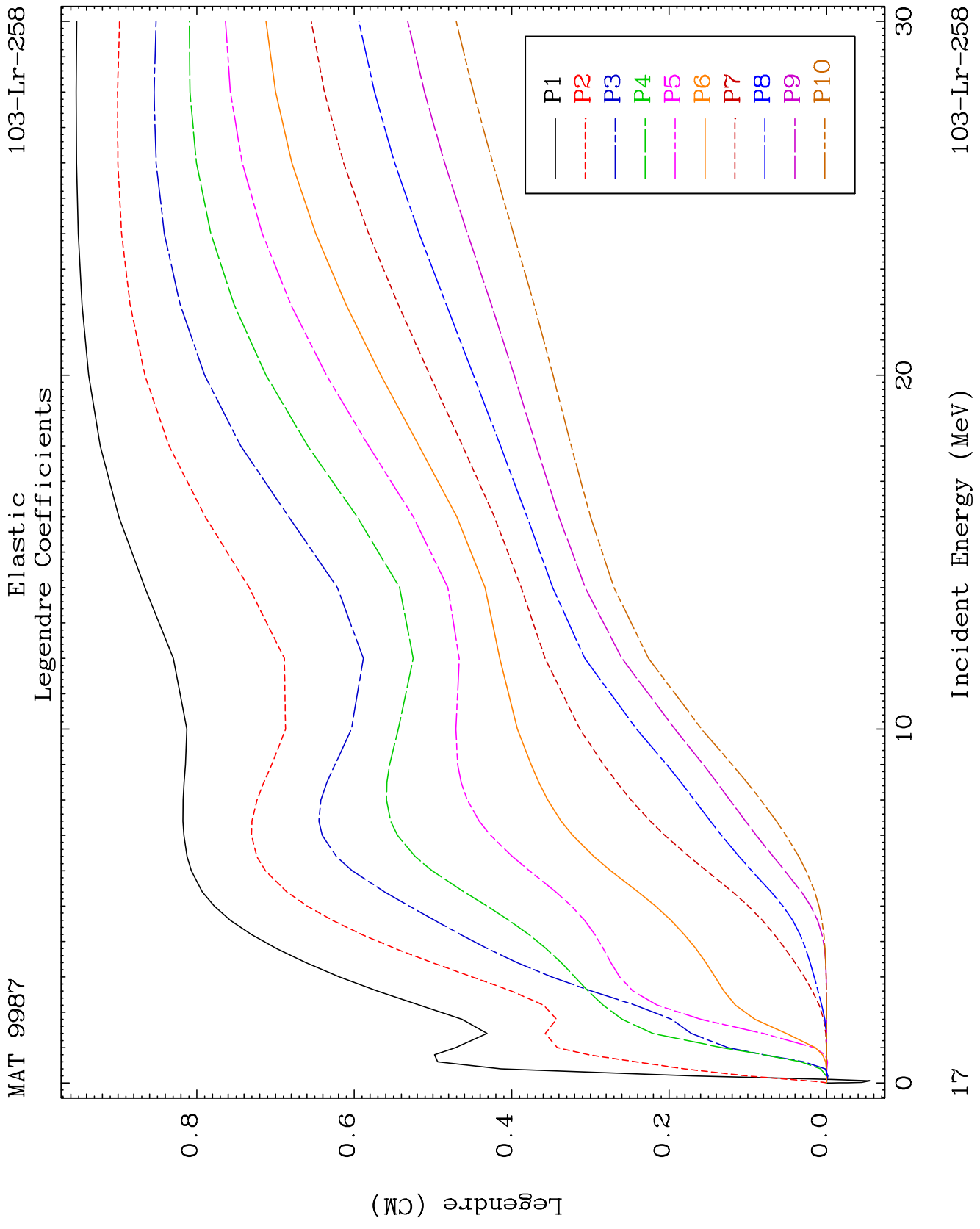
(n,α) Levels
293 Kelvin Cross Sections

103-Lr-258





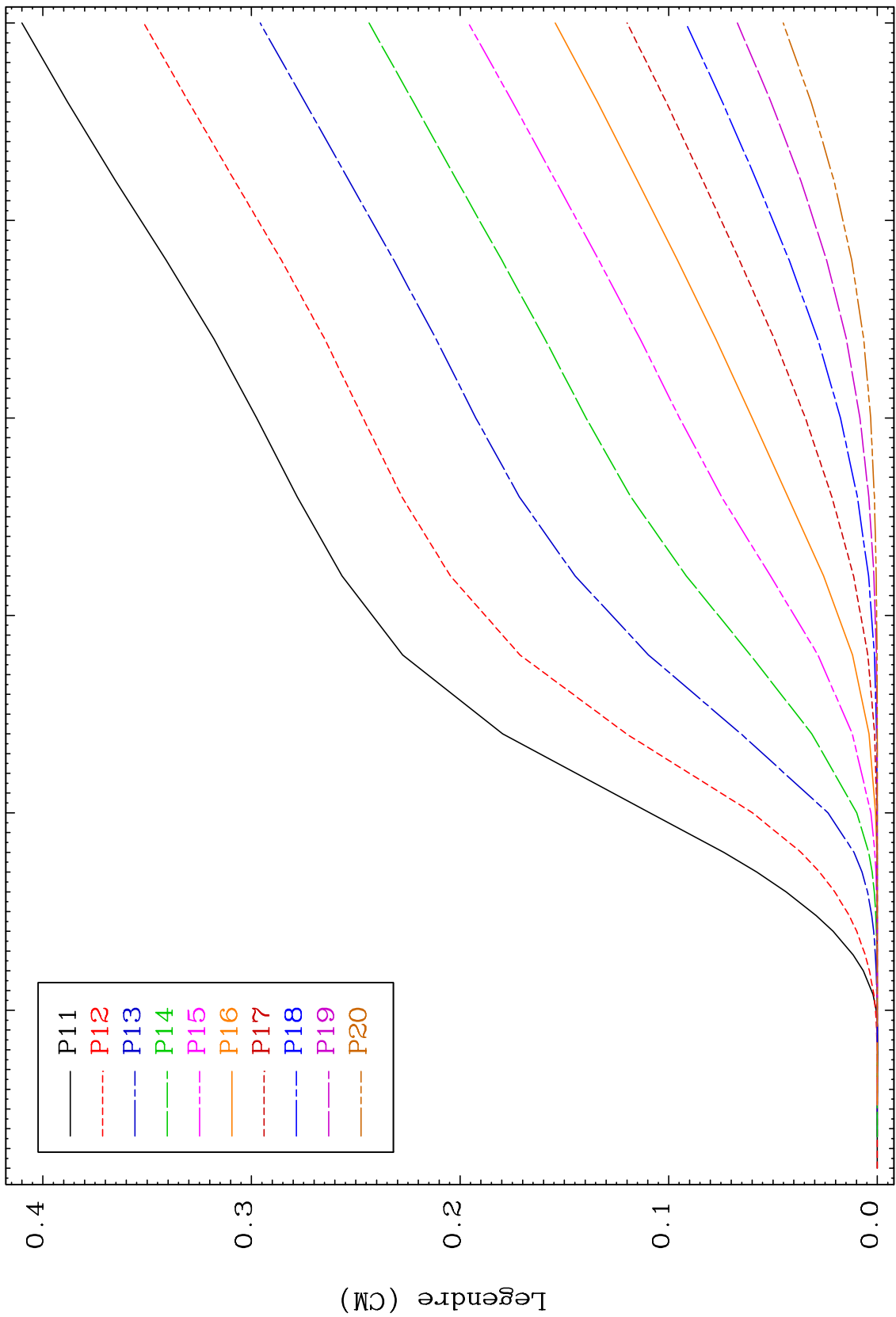




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Elastic Legendre Coefficients

103-Lr-258



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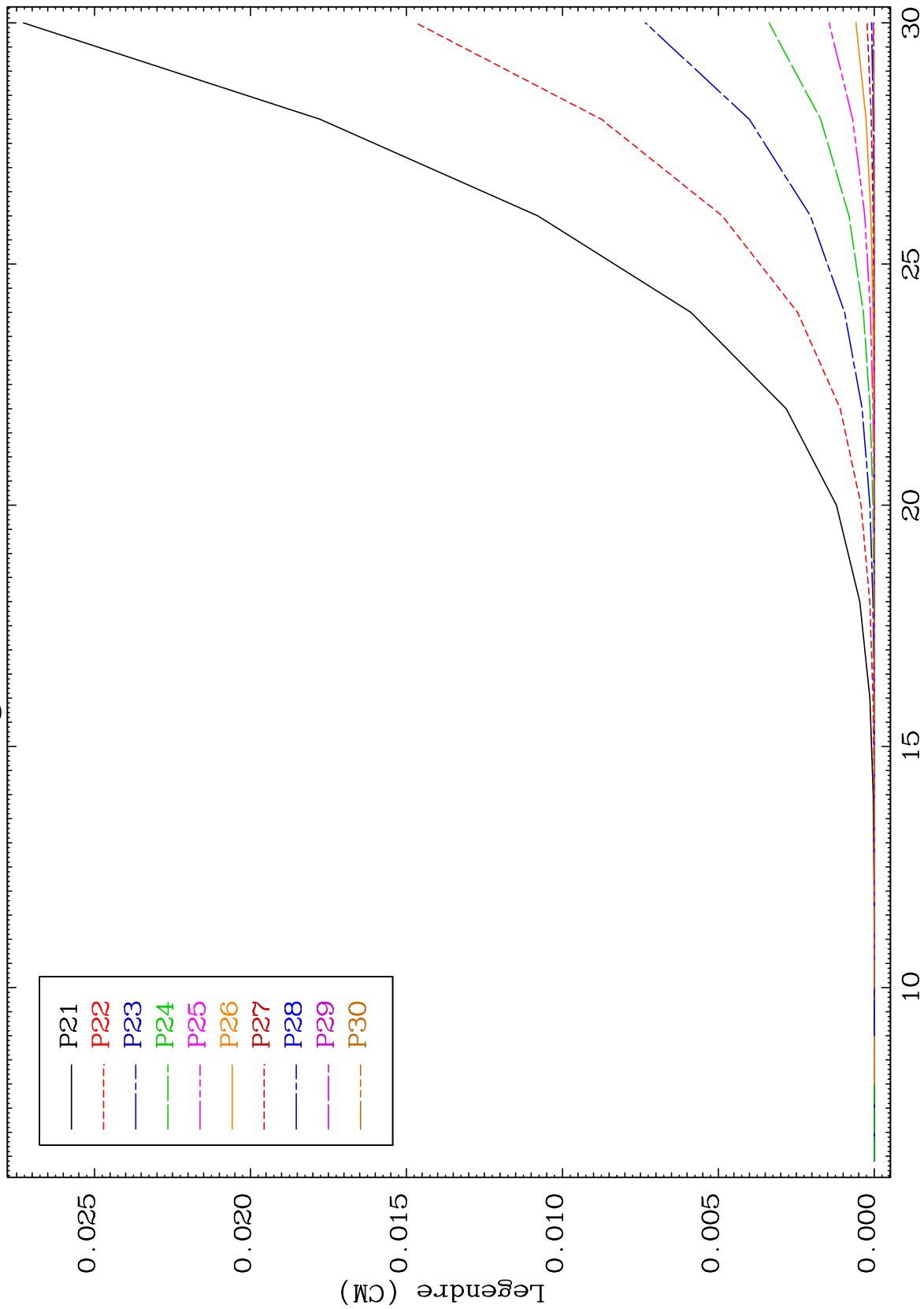
Incident Energy (MeV)

103-Lr-258

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Elastic Legendre Coefficients

103-Lr-258



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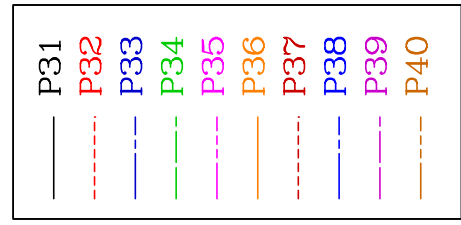
Incident Energy (MeV)

103-Lr-258

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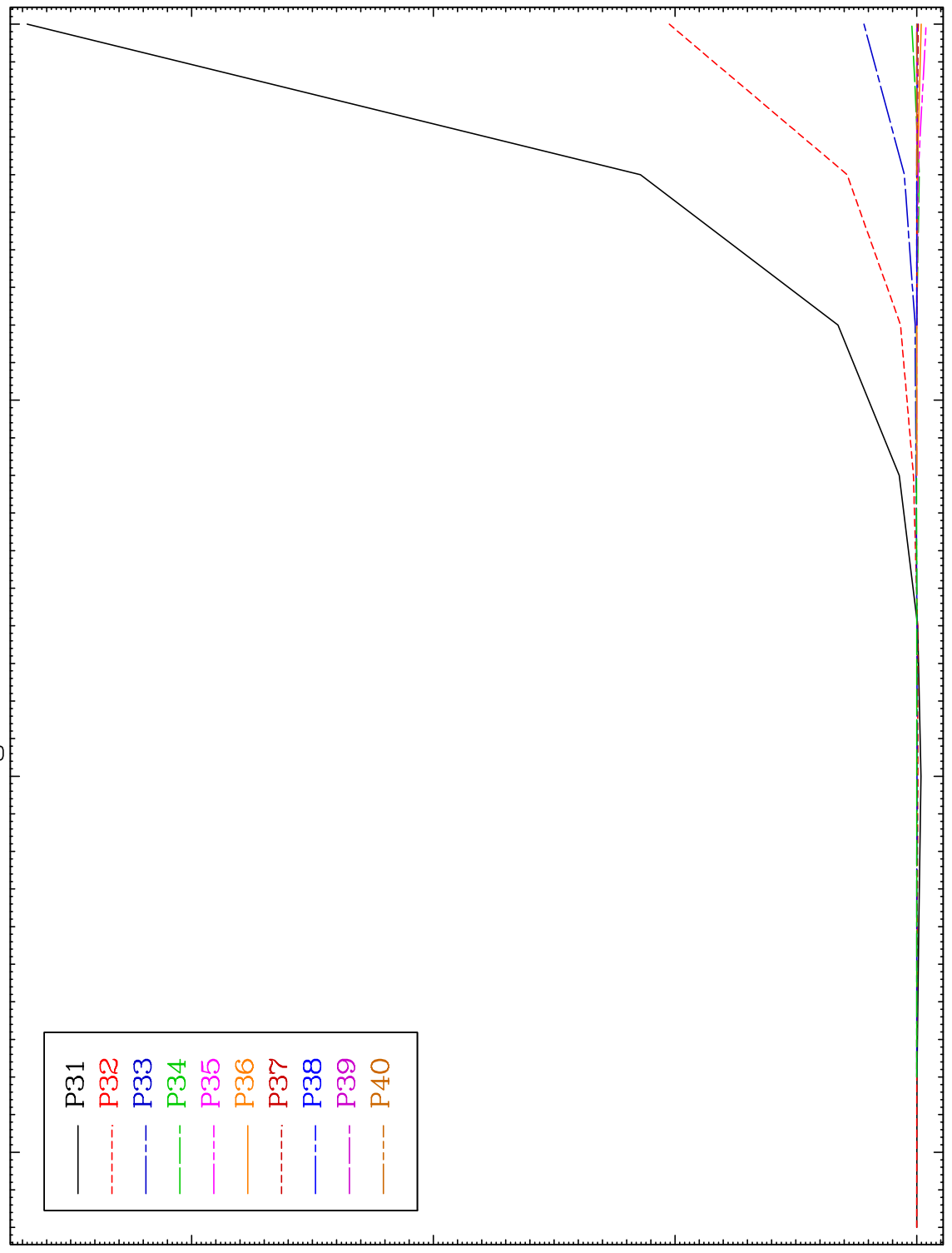
Elastic Legendre Coefficients

103-Lr-258



$\times 10^{-6}$

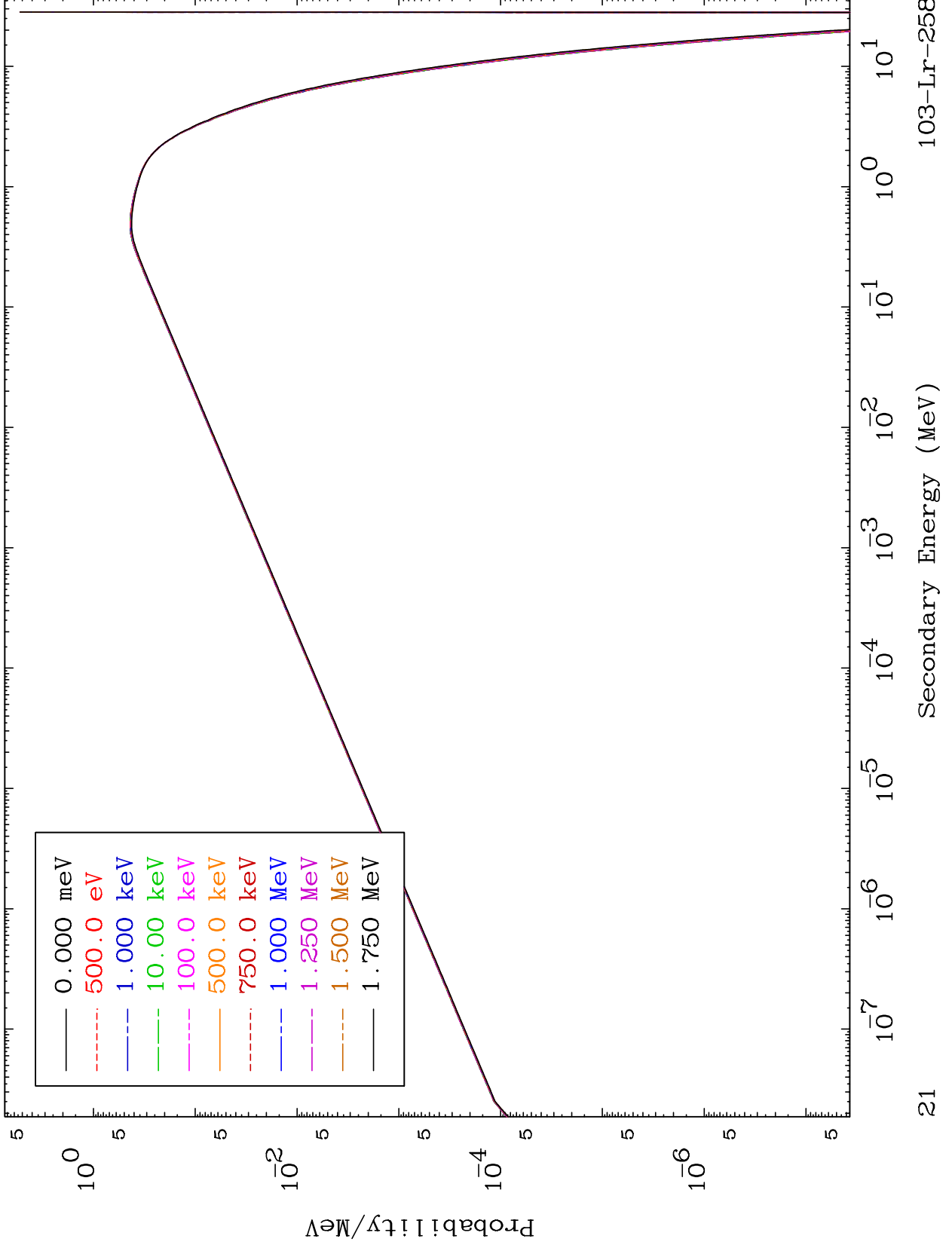
Legendre (CM)



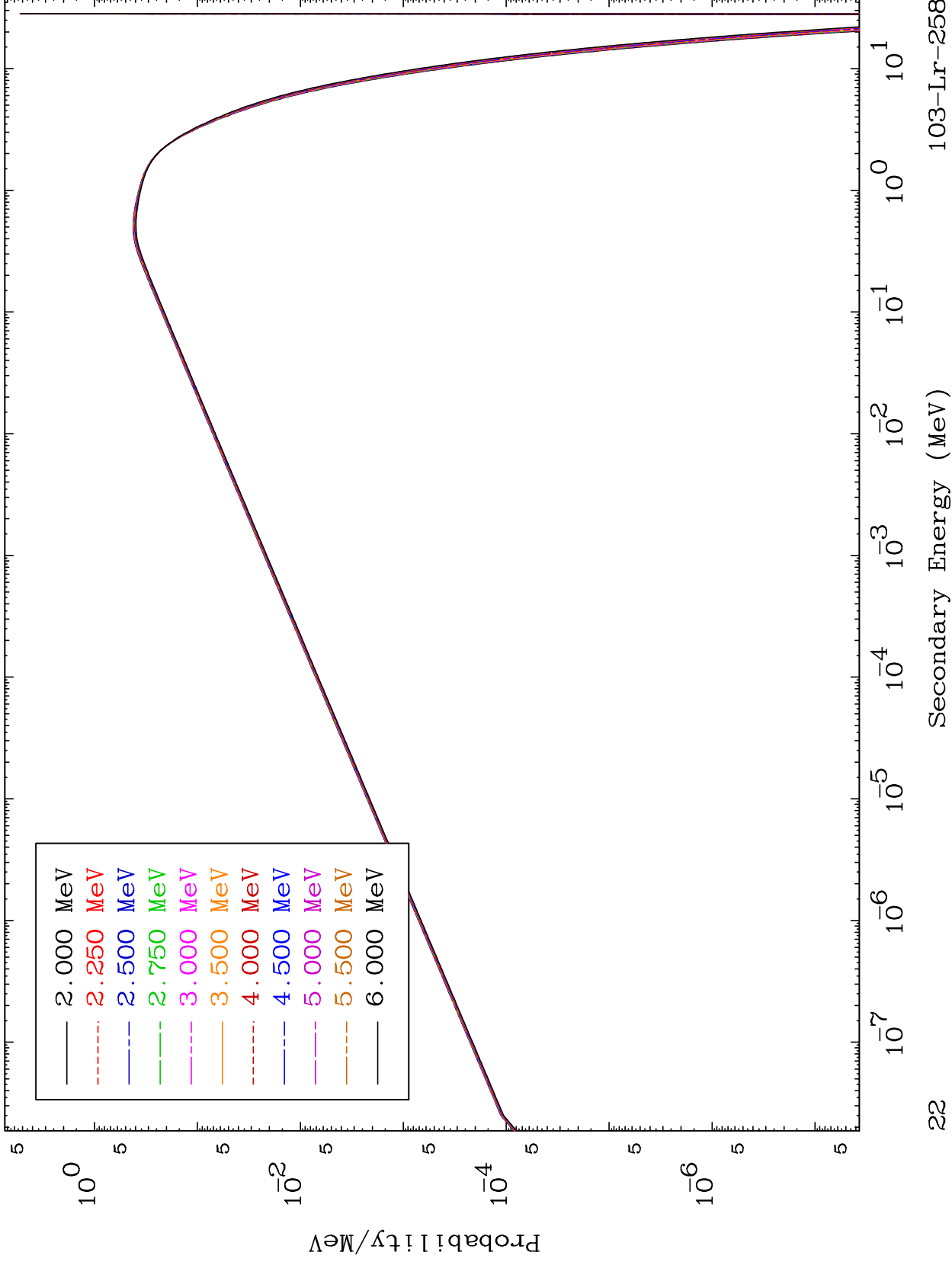
20

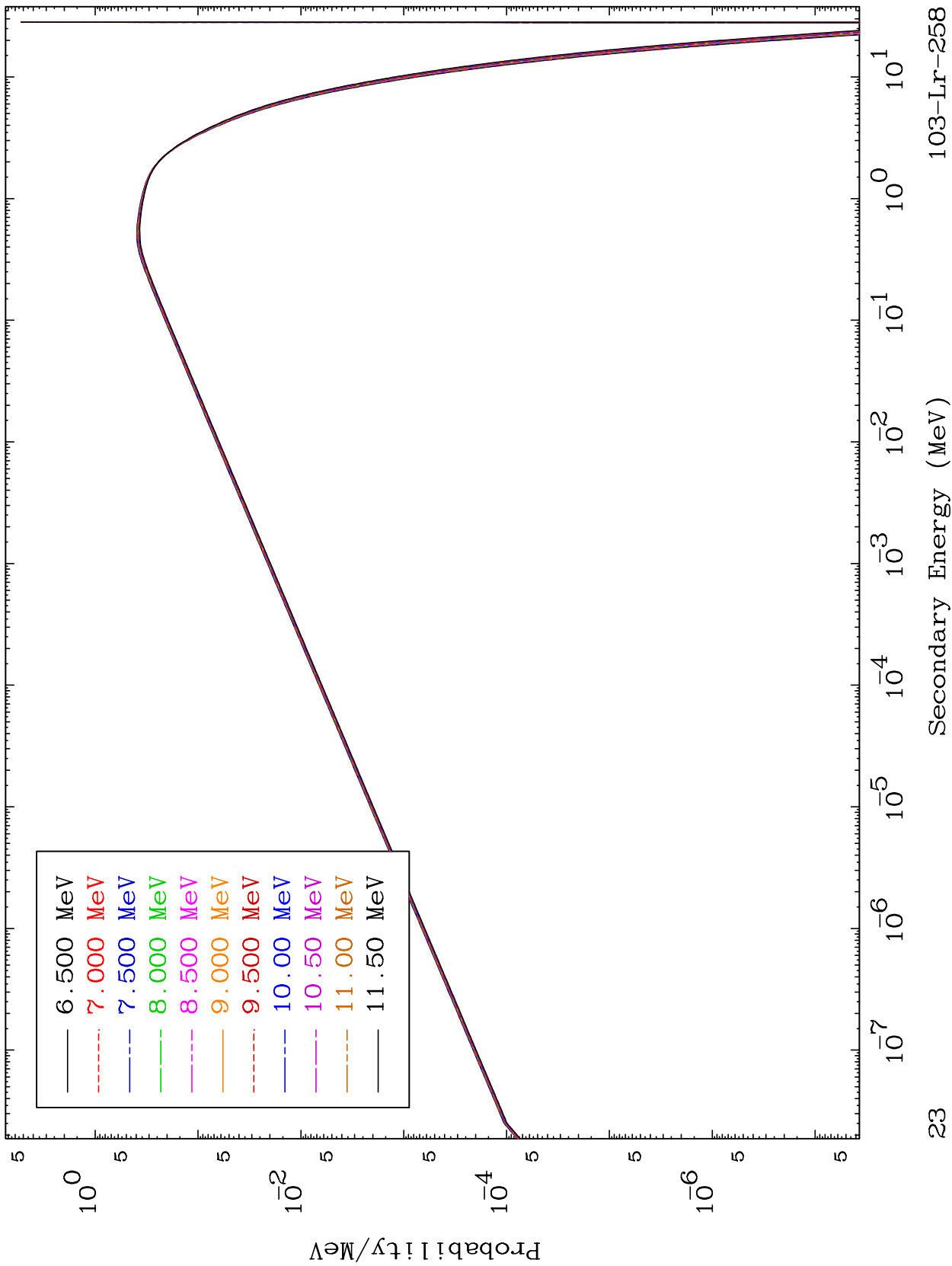
Incident Energy (MeV)

103-Lr-258

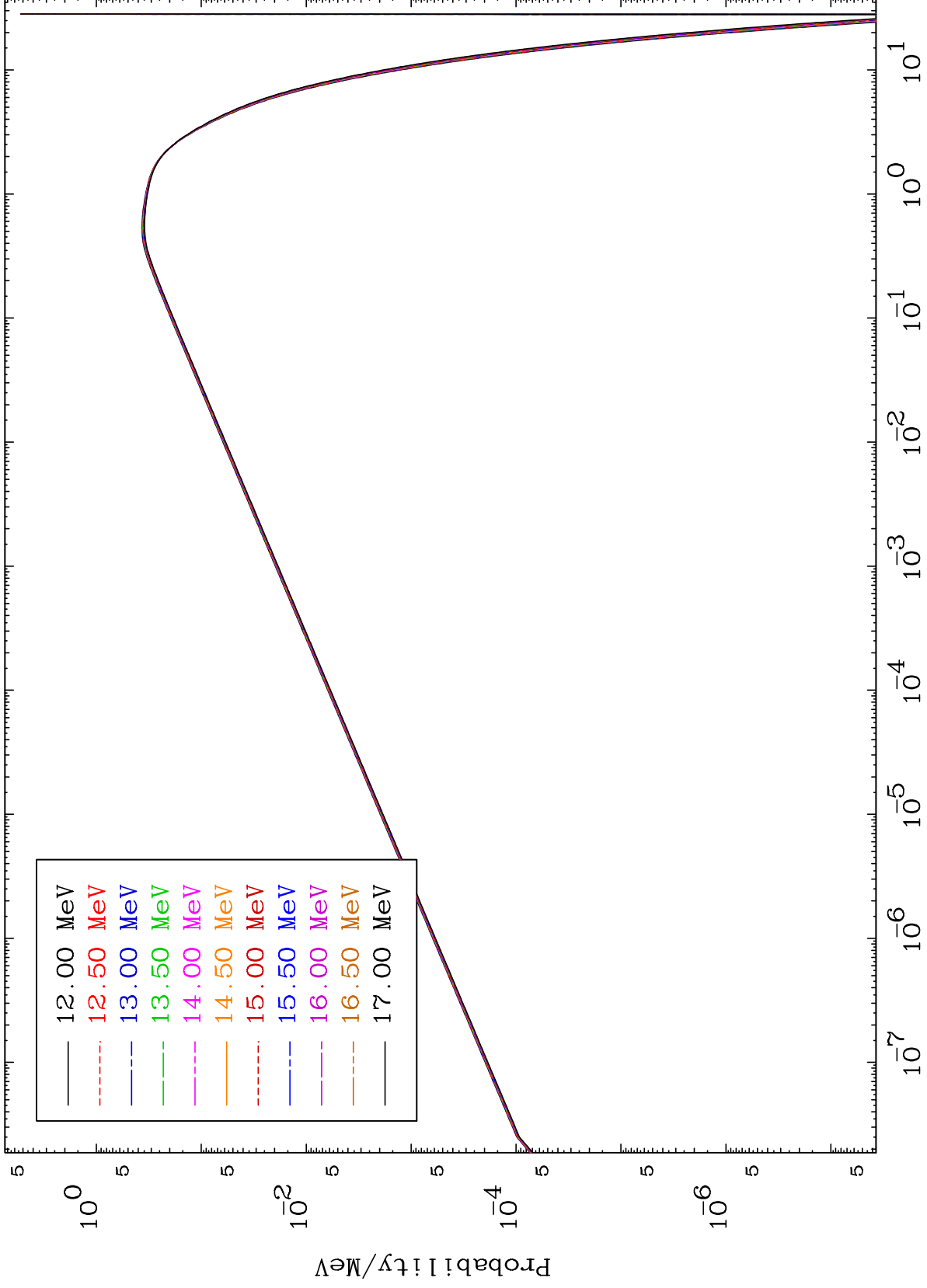


Fission
Energy Distribution

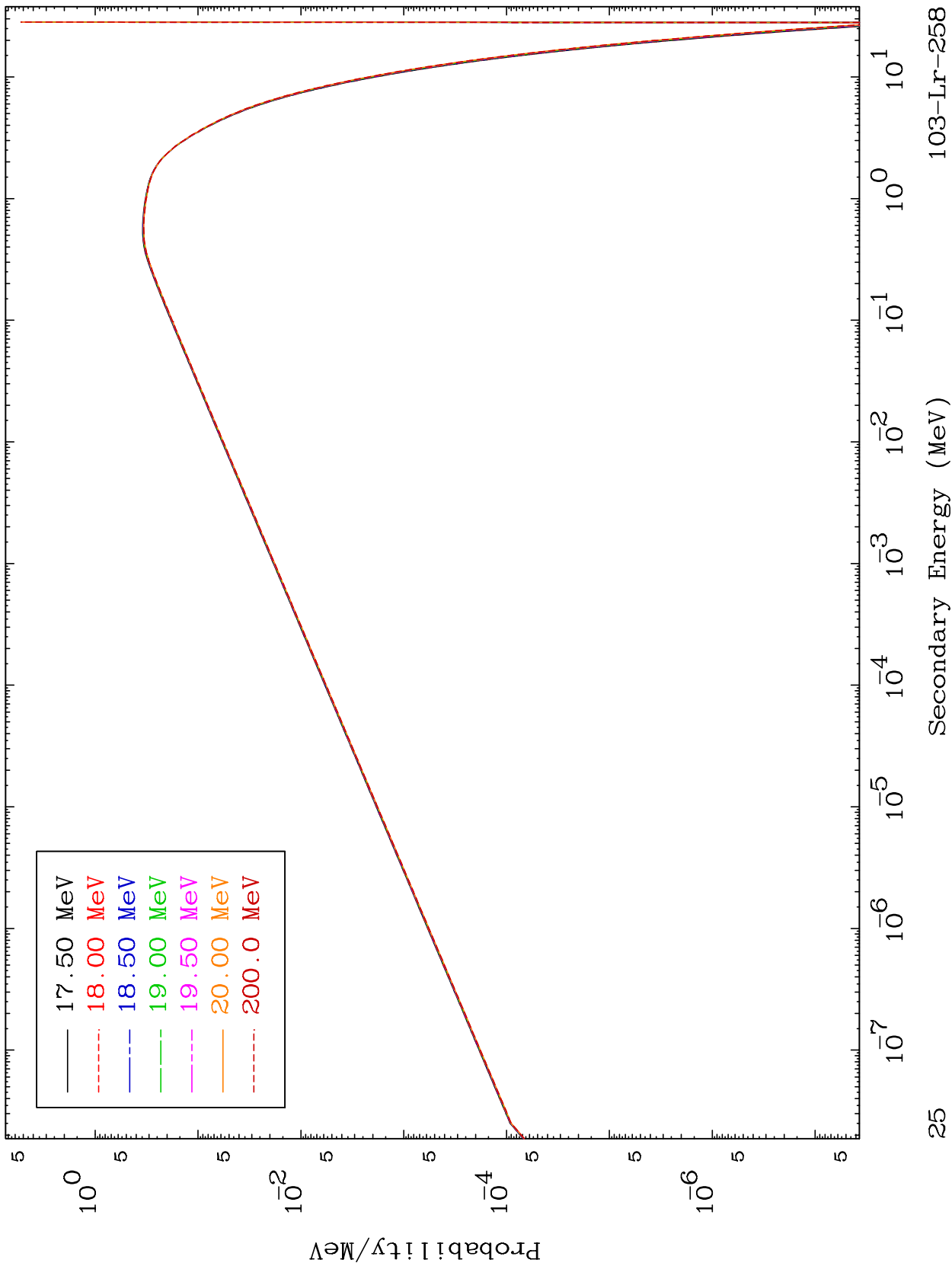




Fission
Energy Distribution



Fission
Energy Distribution

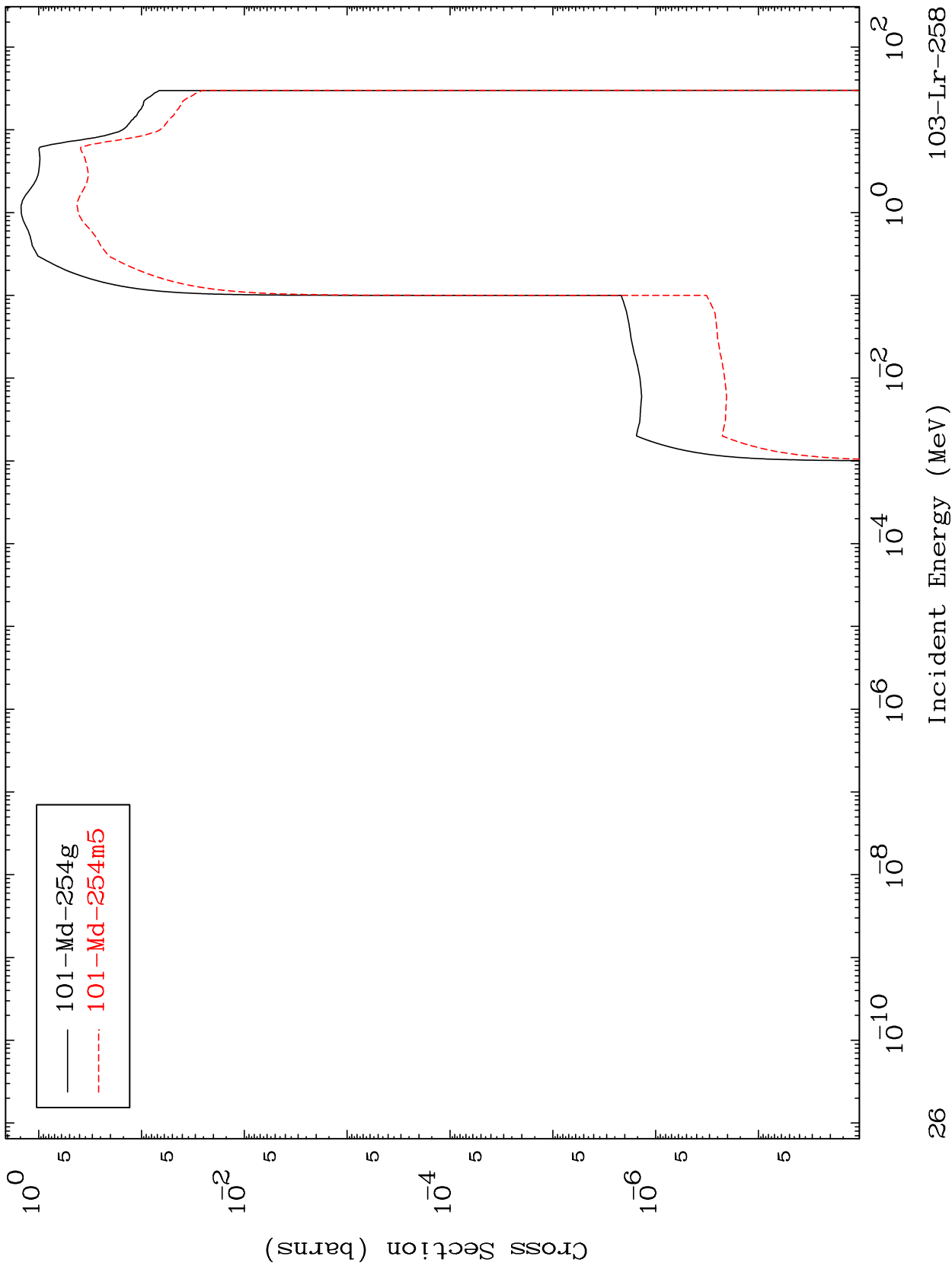


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$(n, n') \alpha$

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Radionuclide Production Cross Section

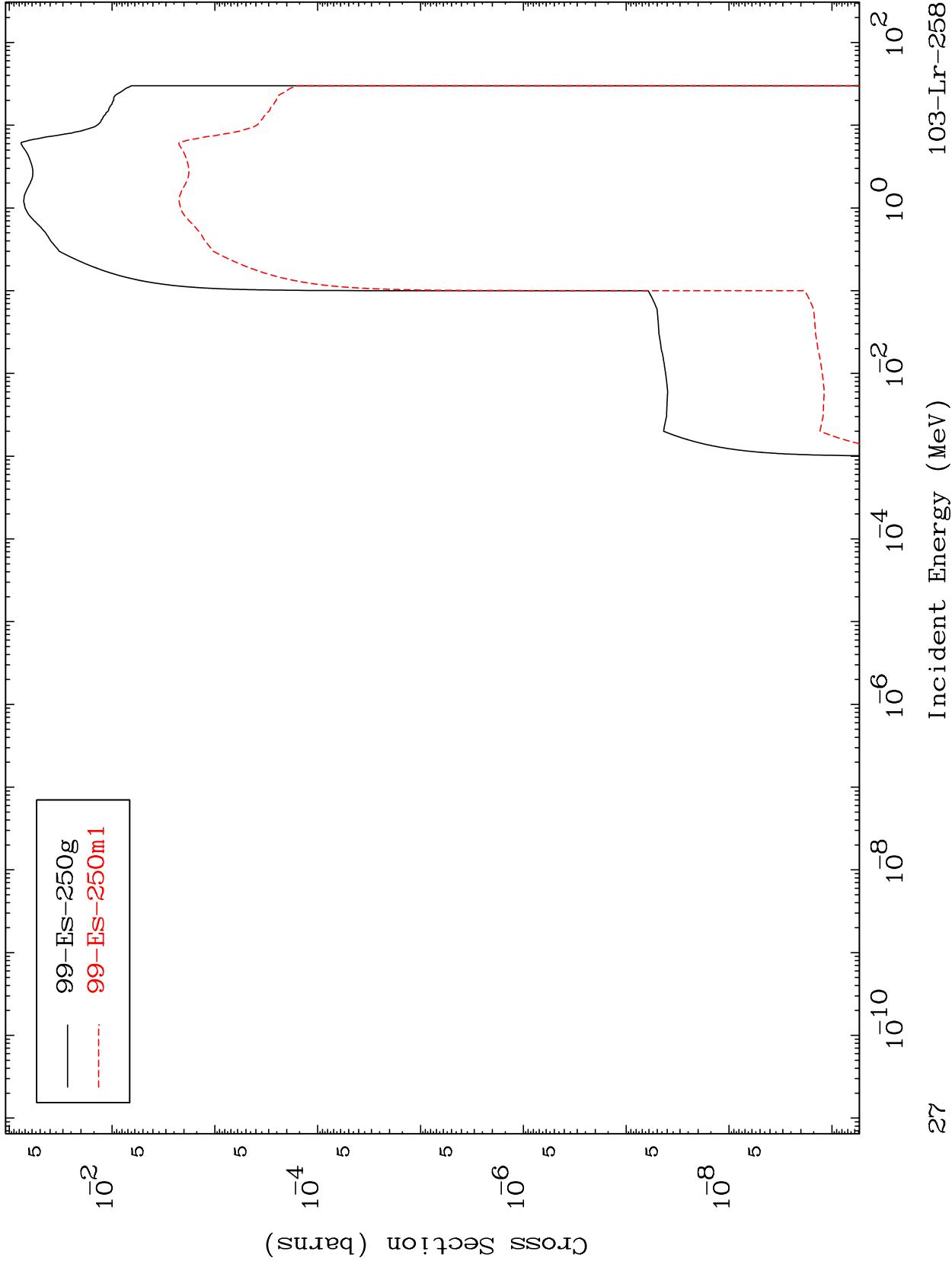


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(n,n') 2α

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Radionuclide Production Cross Section

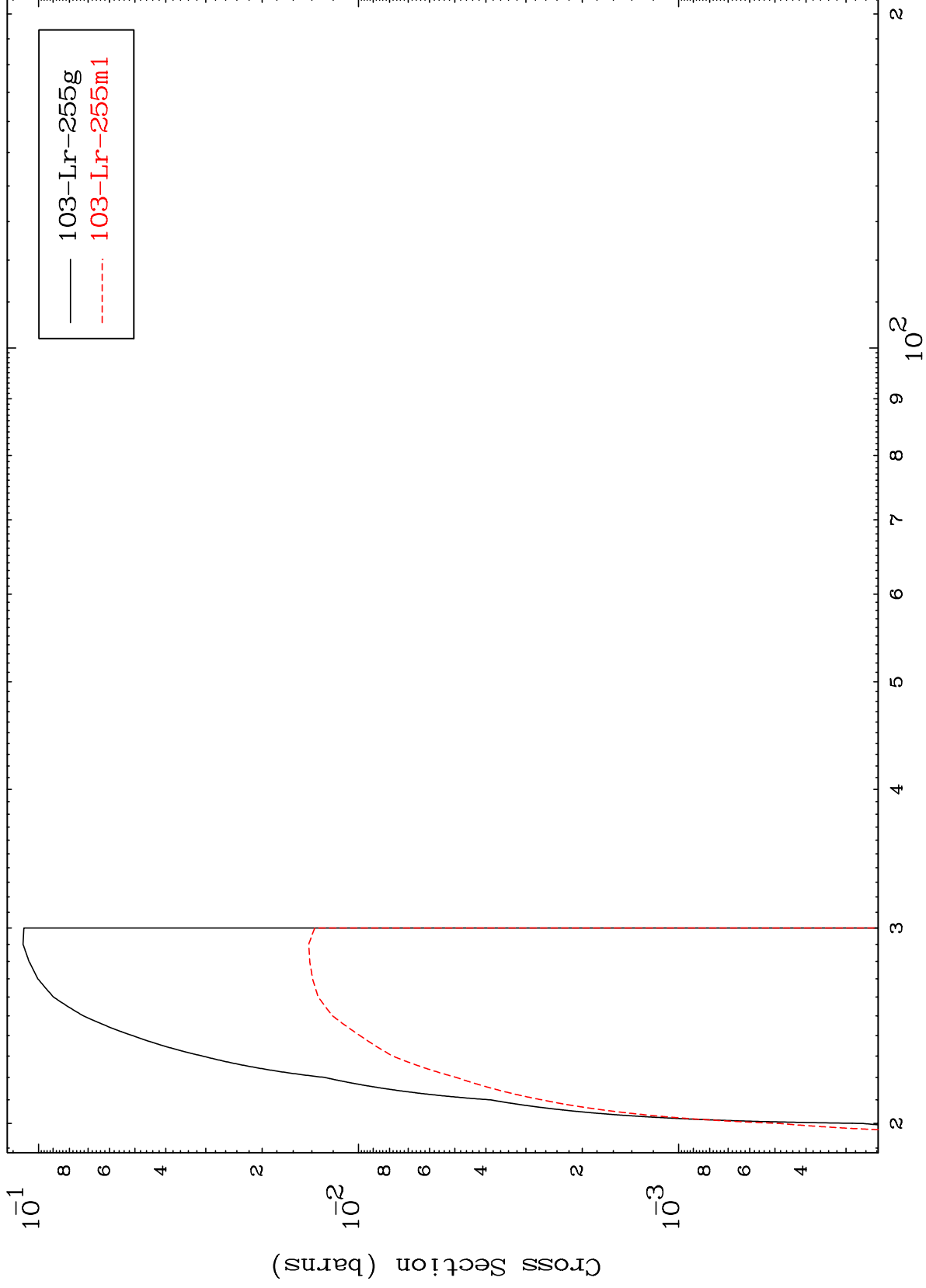


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(n,4n)

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Radionuclide Production Cross Section



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Incident Energy (MeV)

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