

Program EVALPLOT
(Version 2015-2)

by

Dermott E. Cullen
(Present Contact Information)

Dermott E. Cullen
1466 Hudson Way
Livermore, CA 94550
U.S.A.

Tele: 925-443-1911

E.Mail:redcullen1@comcast.net
Web:home.comcast.net/~redcullen1

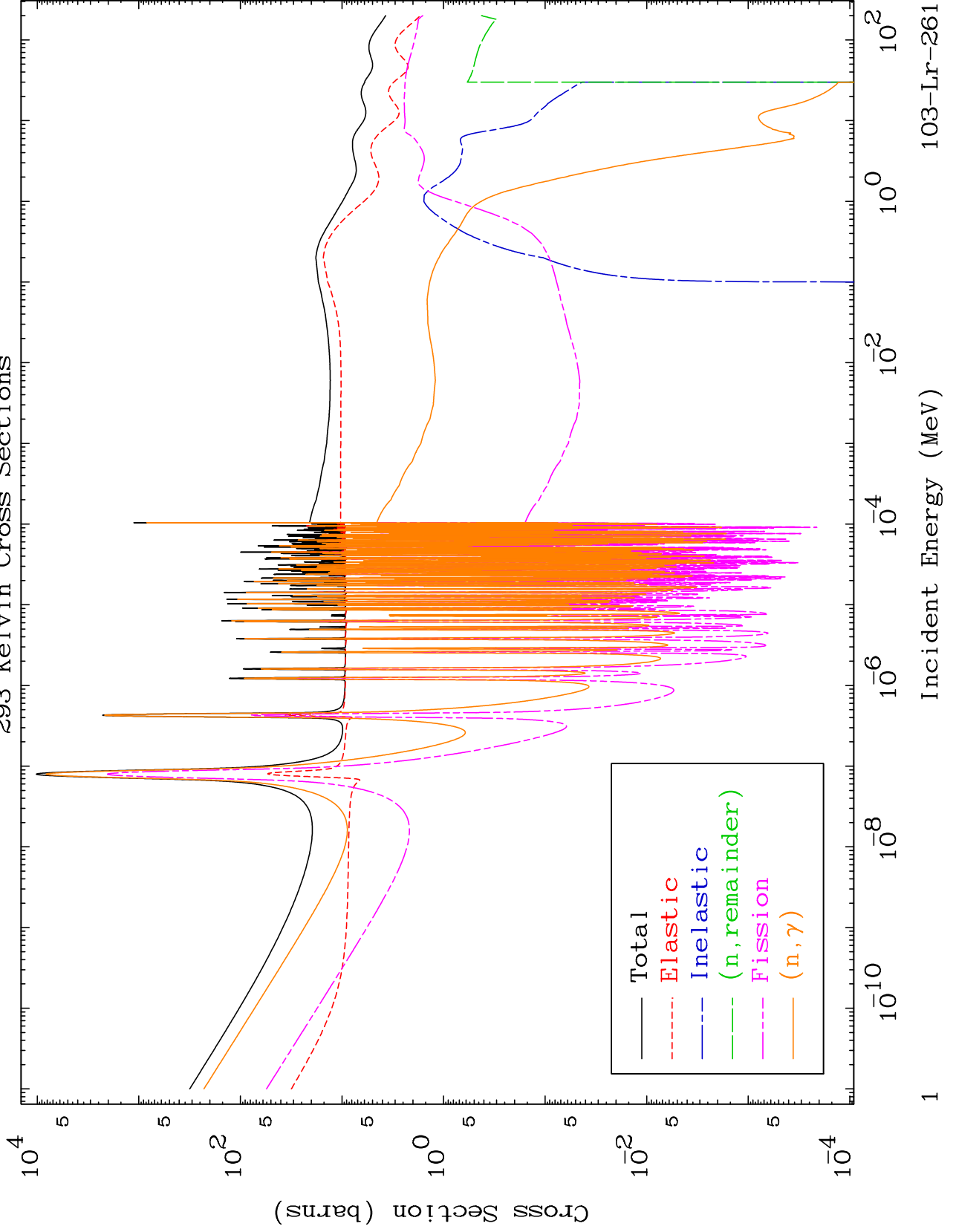
Press Mouse Button to Start

MAT 9990

Major

293 Kelvin Cross Sections

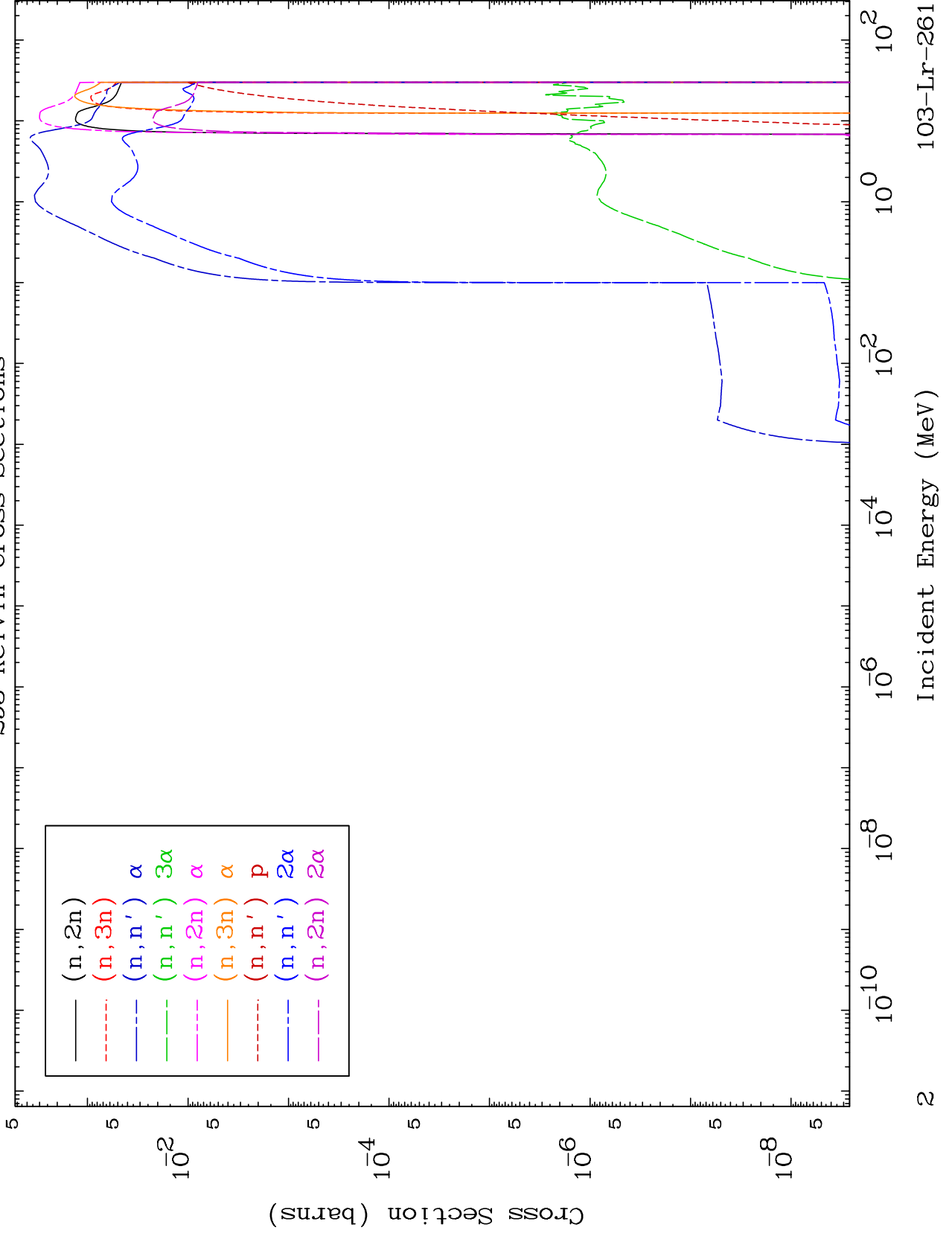
103-Lr-261



MAT 9990

Neutron Production
293 Kelvin Cross Sections

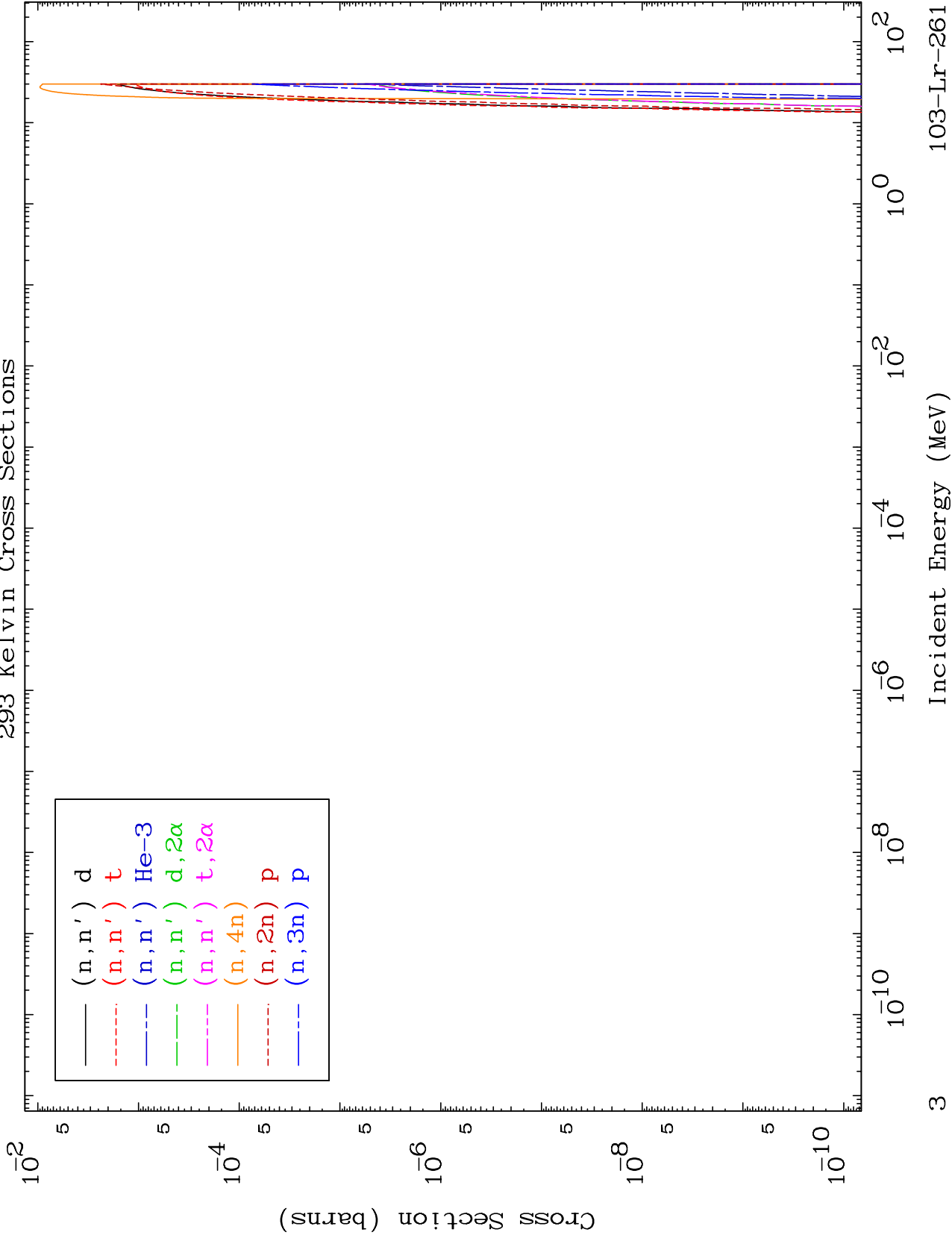
103-Lr-261



MAT 9990

Neutron Production
293 Kelvin Cross Sections

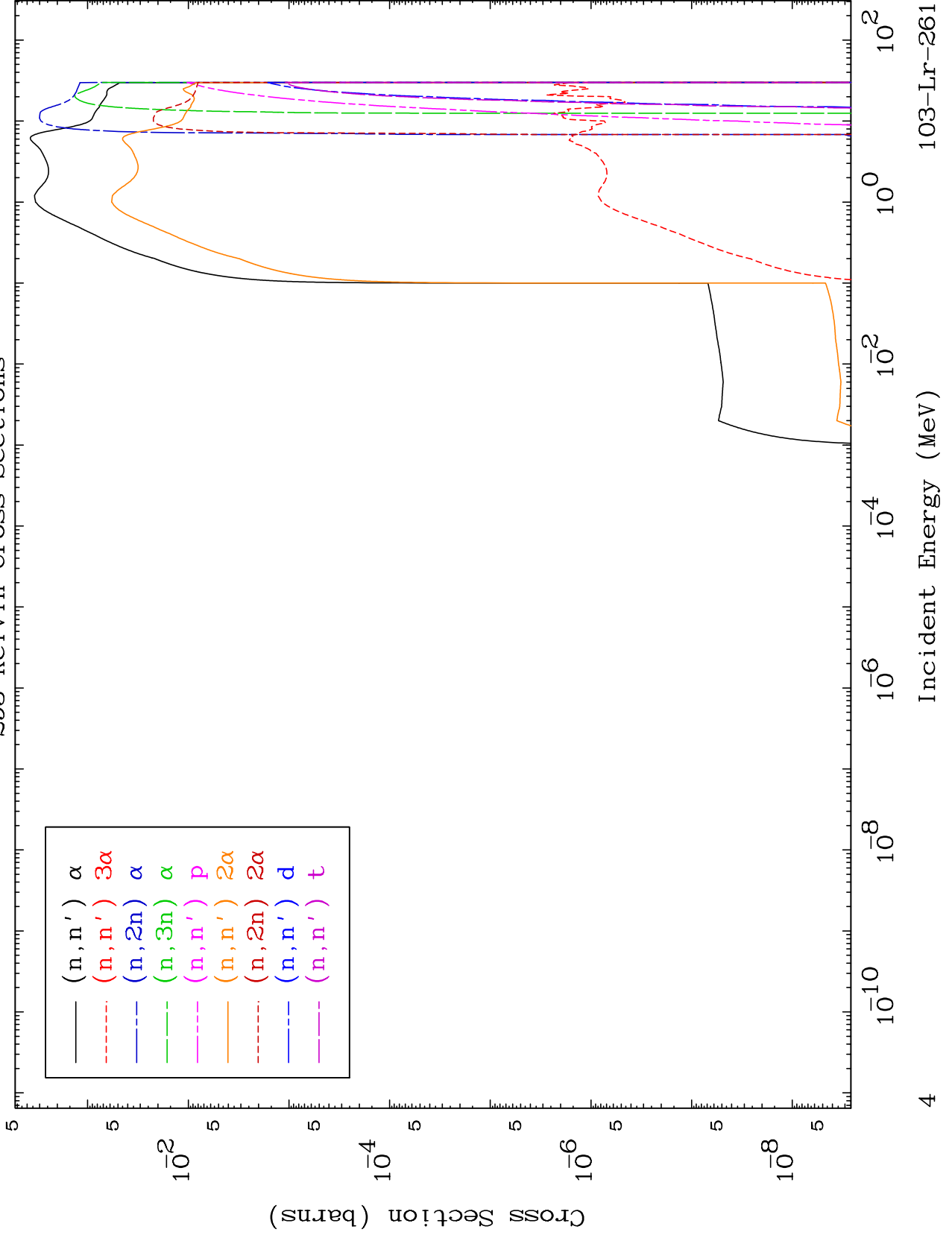
103-Lr-261



MAT 9990

Charged Particle
293 Kelvin Cross Sections

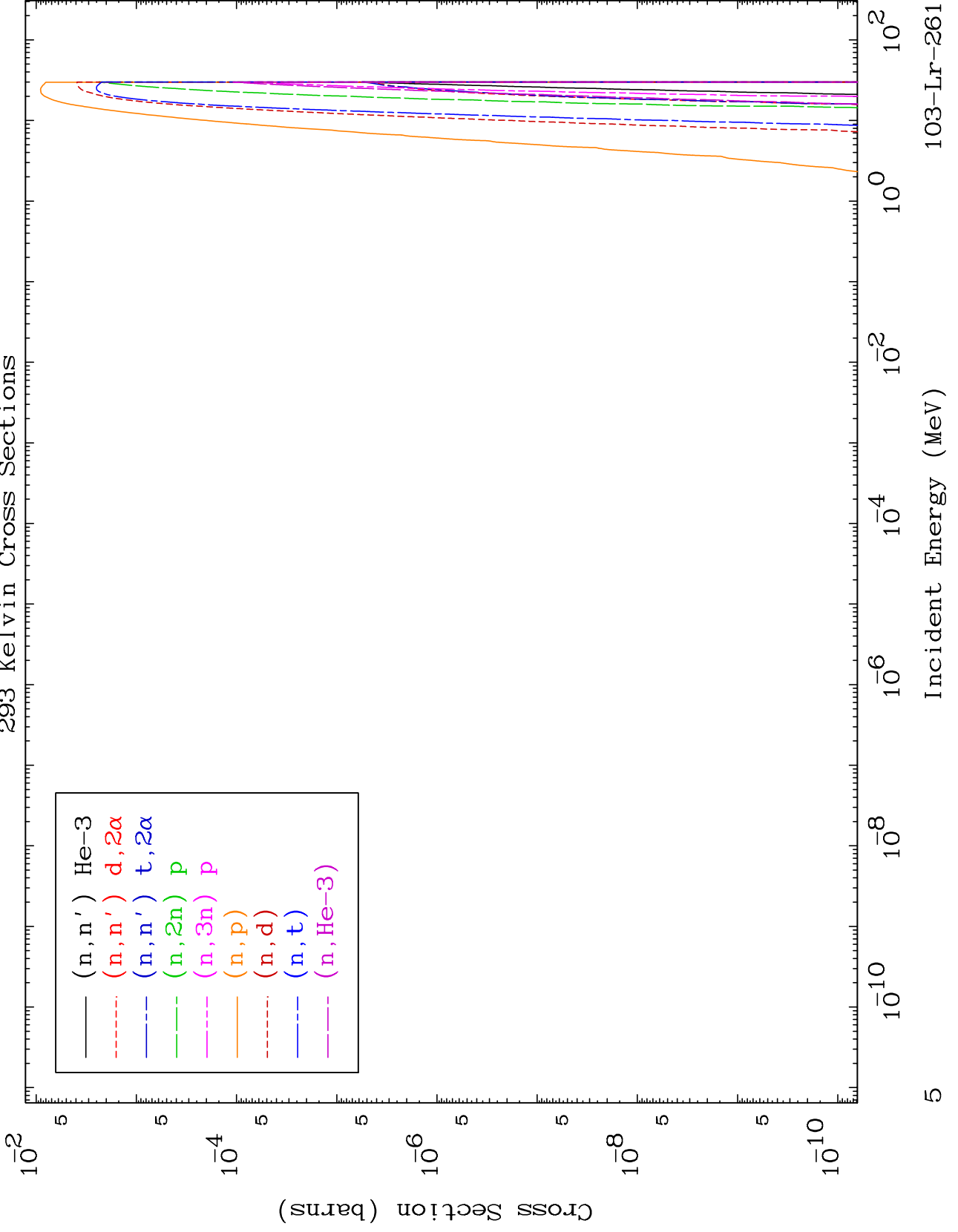
103-Lr-261



MAT 9990

Charged Particle
293 Kelvin Cross Sections

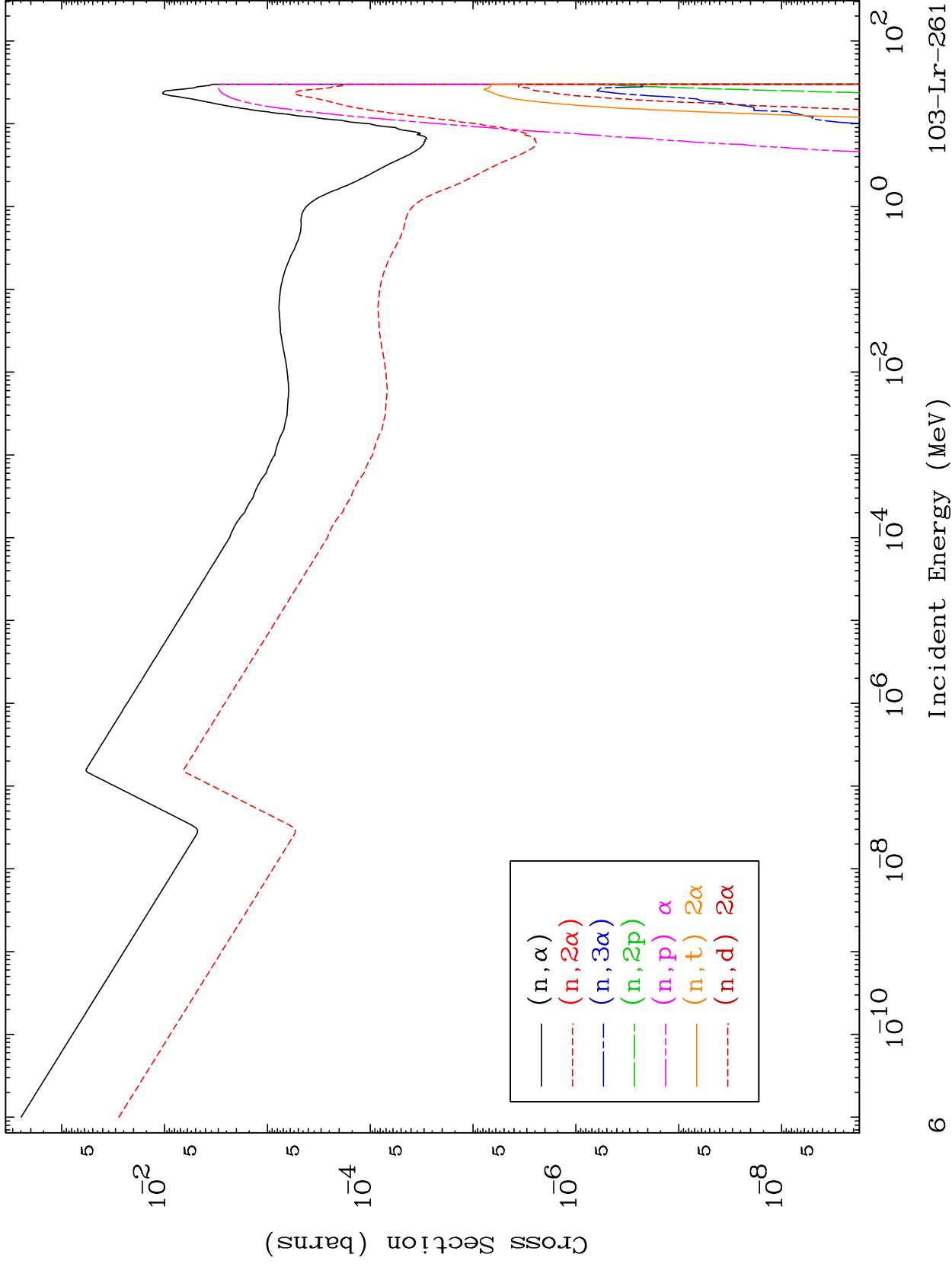
103-Lr-261



MAT 9990

Charged Particle
293 Kelvin Cross Sections

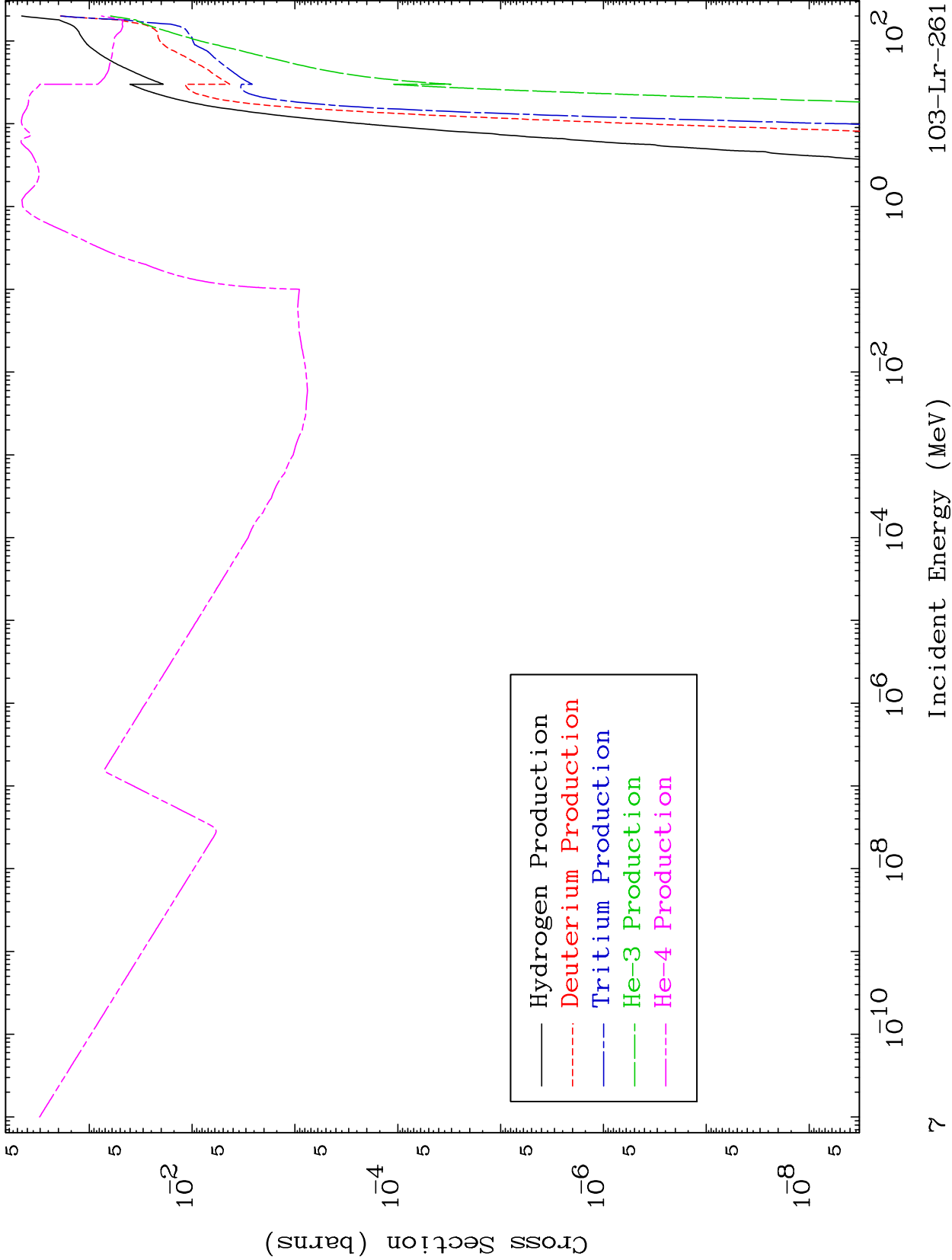
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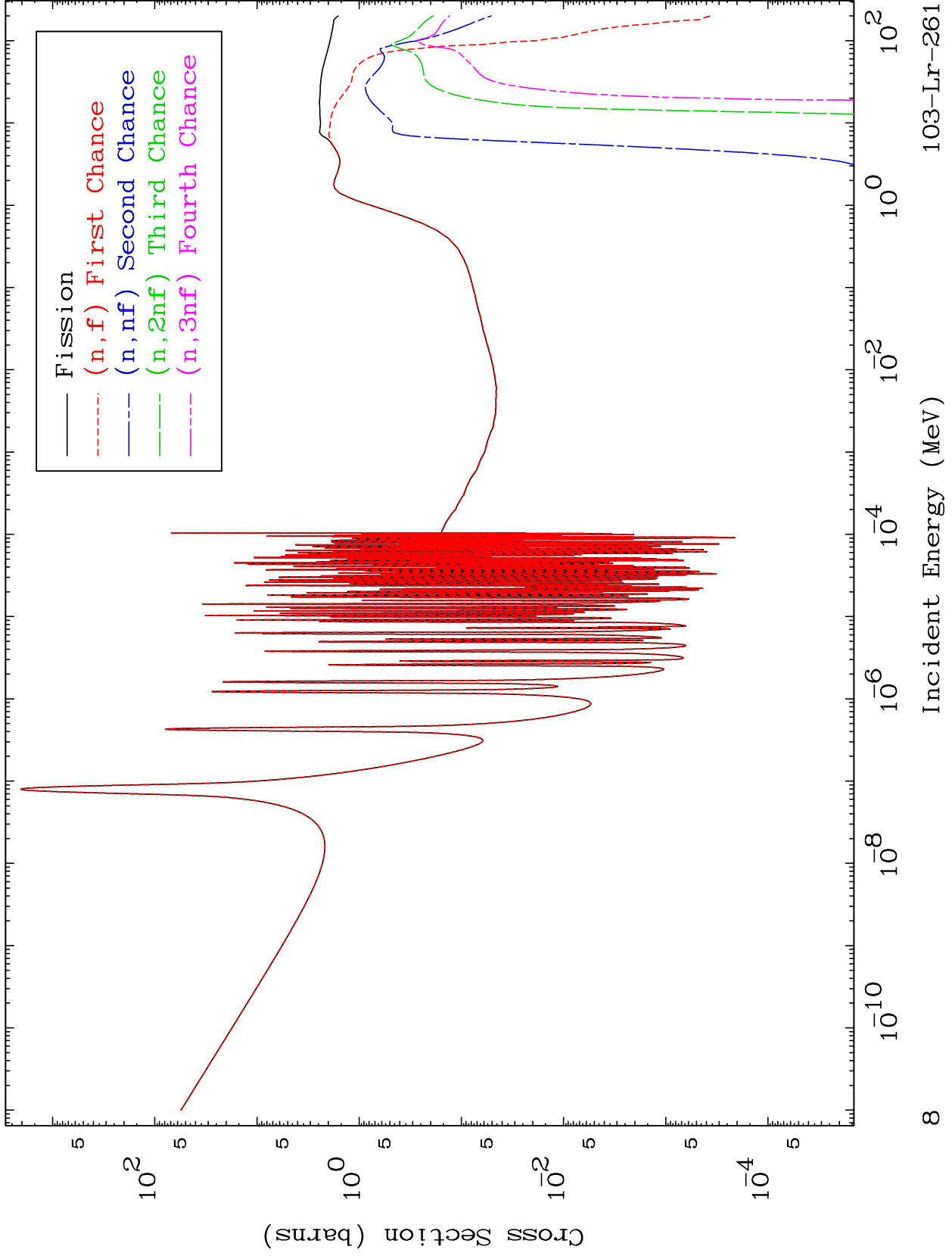


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Particle Production
293 Kelvin Cross Sections

103-Lr-261



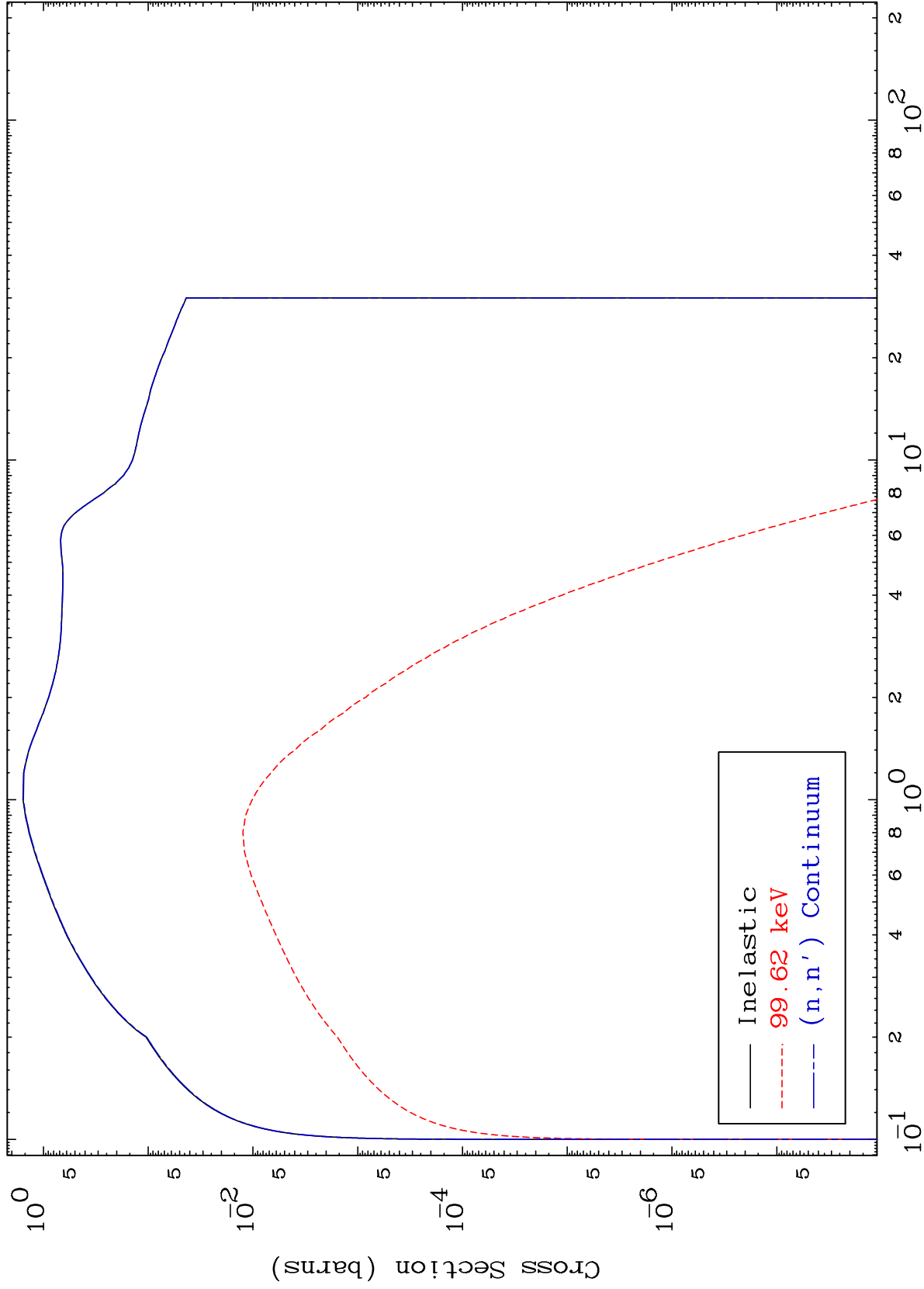


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(n,n') Level

103-Lr-261

293 Kelvin Cross Sections



Incident Energy (MeV)

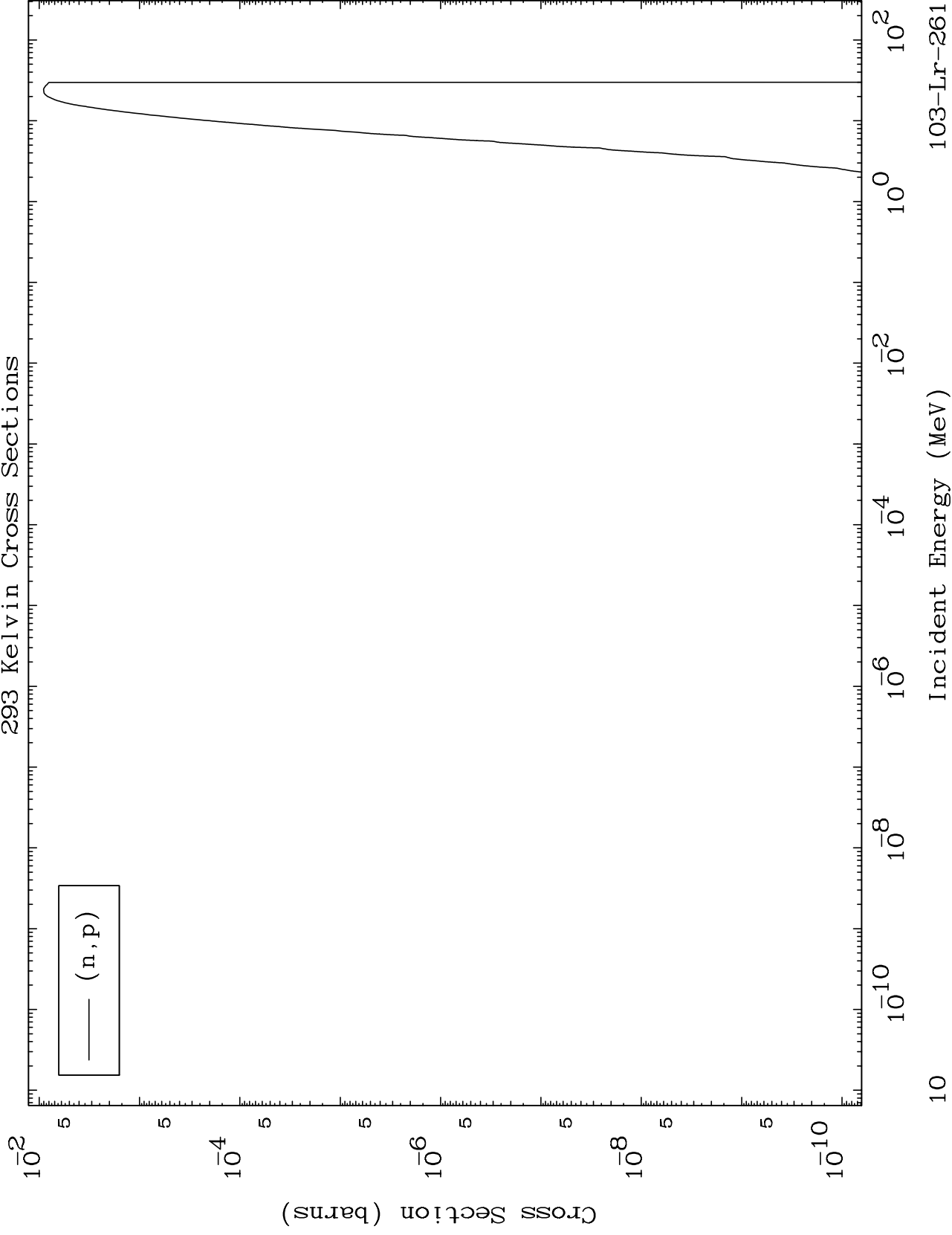
103-Lr-261

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(n,p) Levels
293 Kelvin Cross Sections

103-Lr-261



10

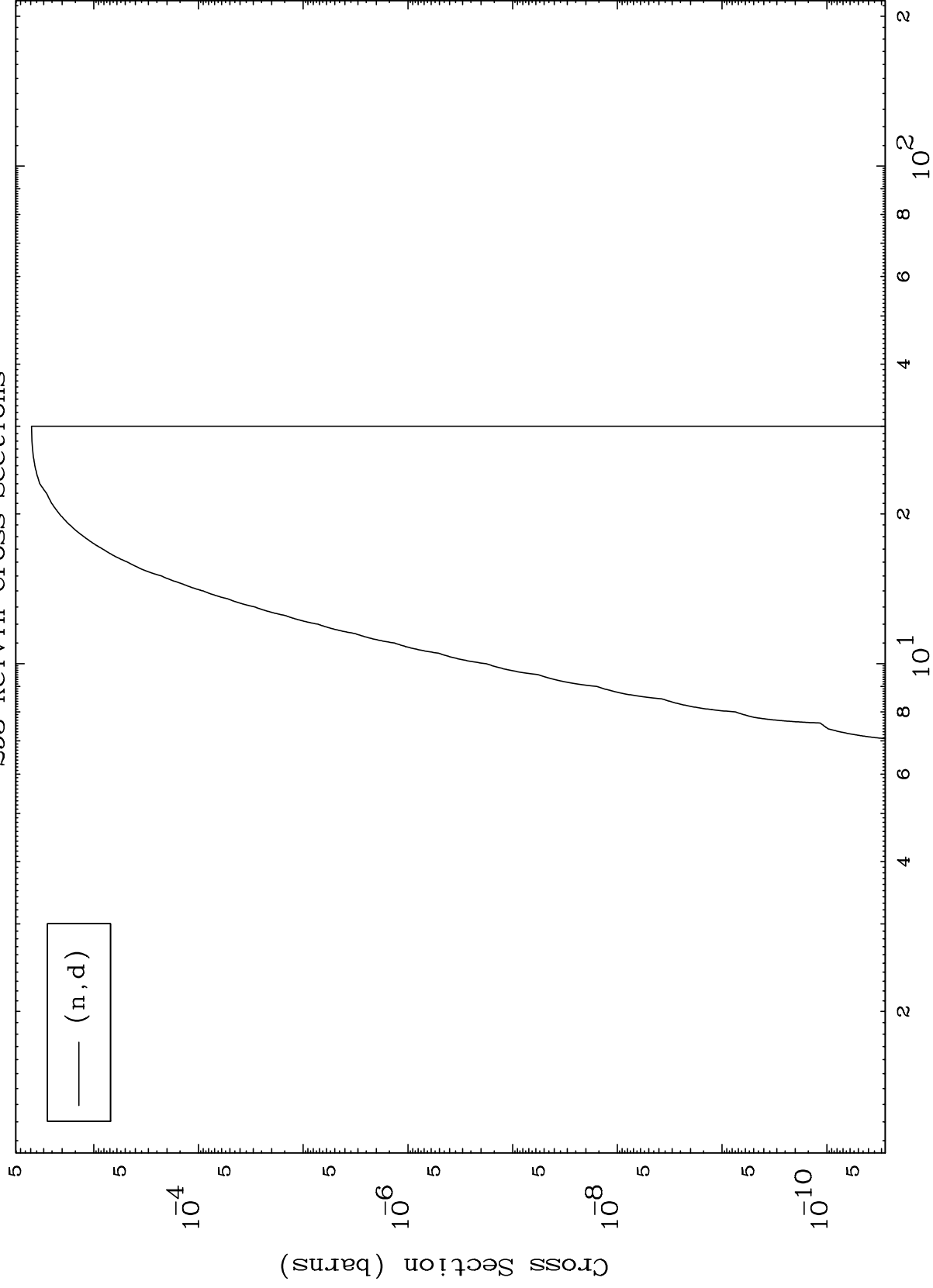
Incident Energy (MeV)

103-Lr-261

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103-Lr-261

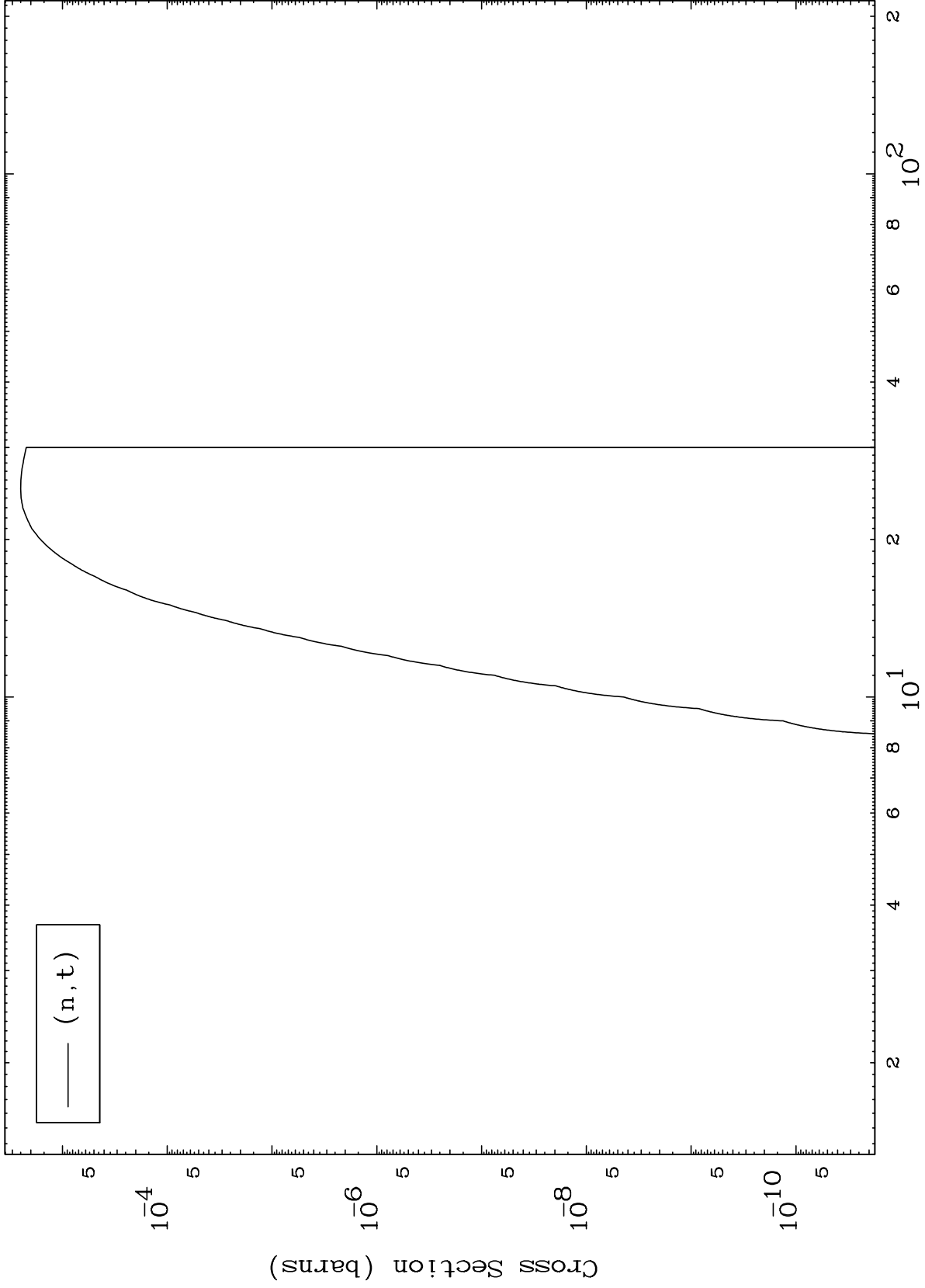
(n,d) Levels
293 Kelvin Cross Sections



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(n,t) Levels
293 Kelvin Cross Sections

103-Lr-261



12

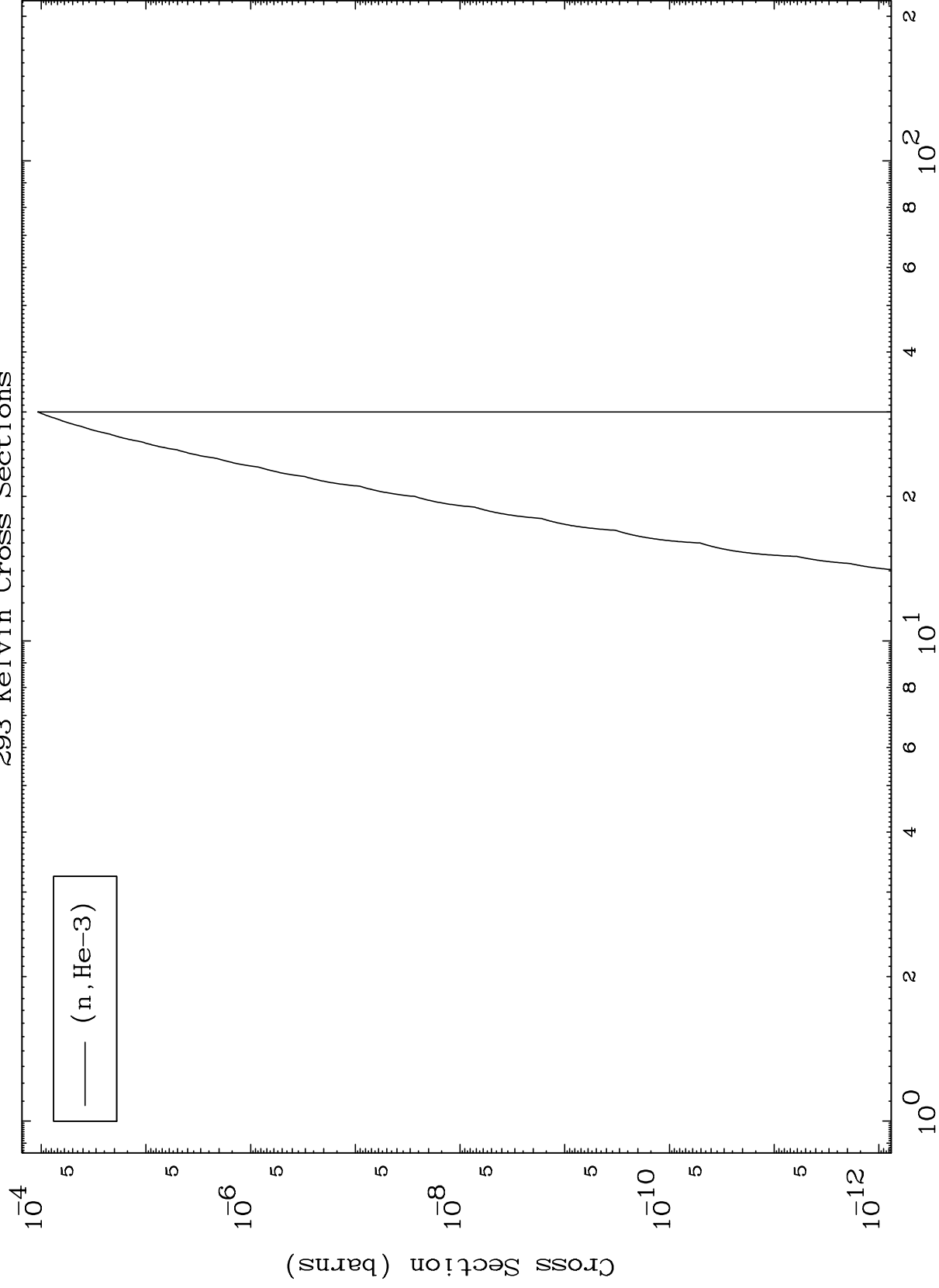
Incident Energy (MeV)

103-Lr-261

MAT 9990

(n,He3) Levels
293 Kelvin Cross Sections

103-Lr-261



Incident Energy (MeV)

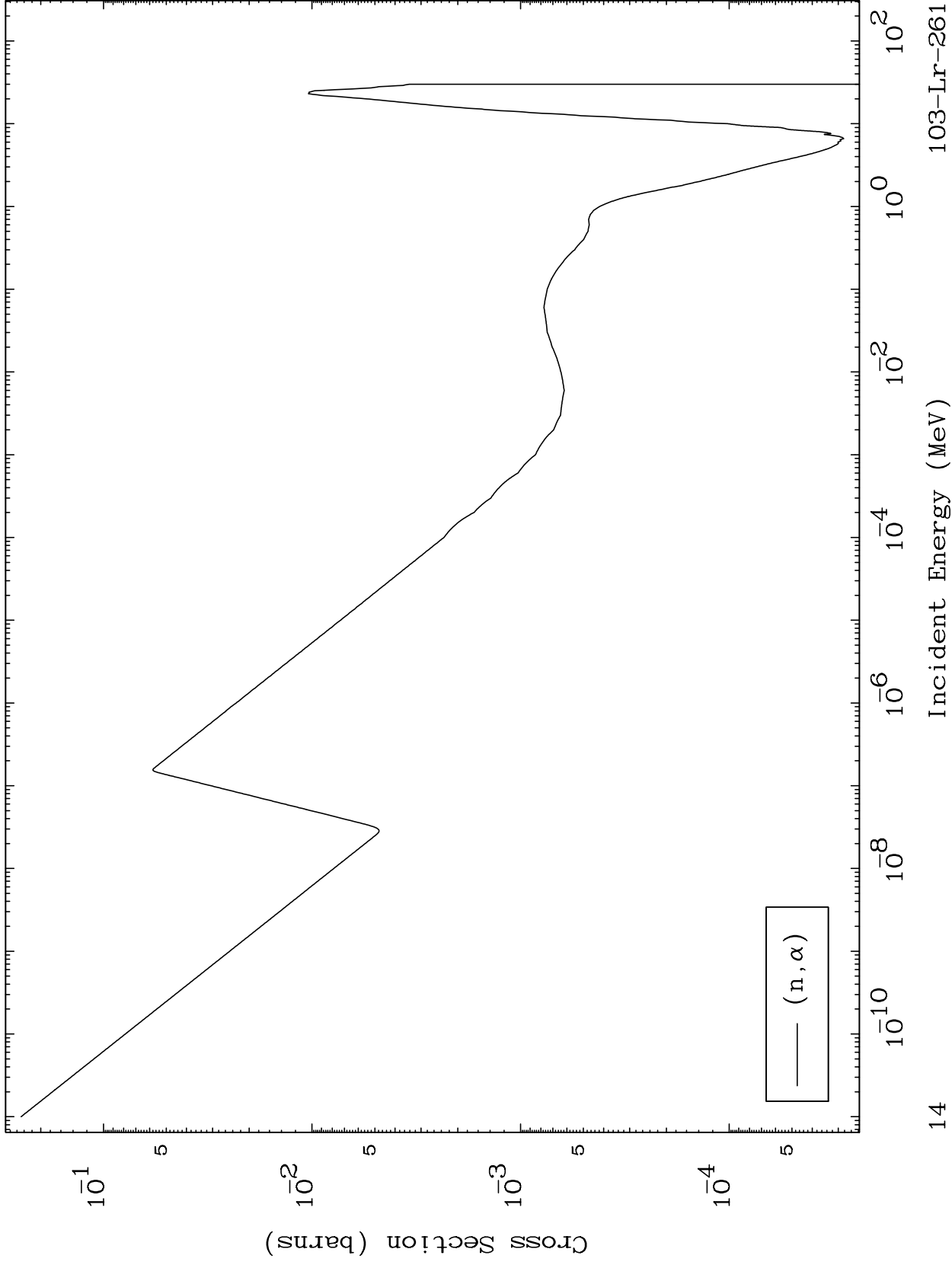
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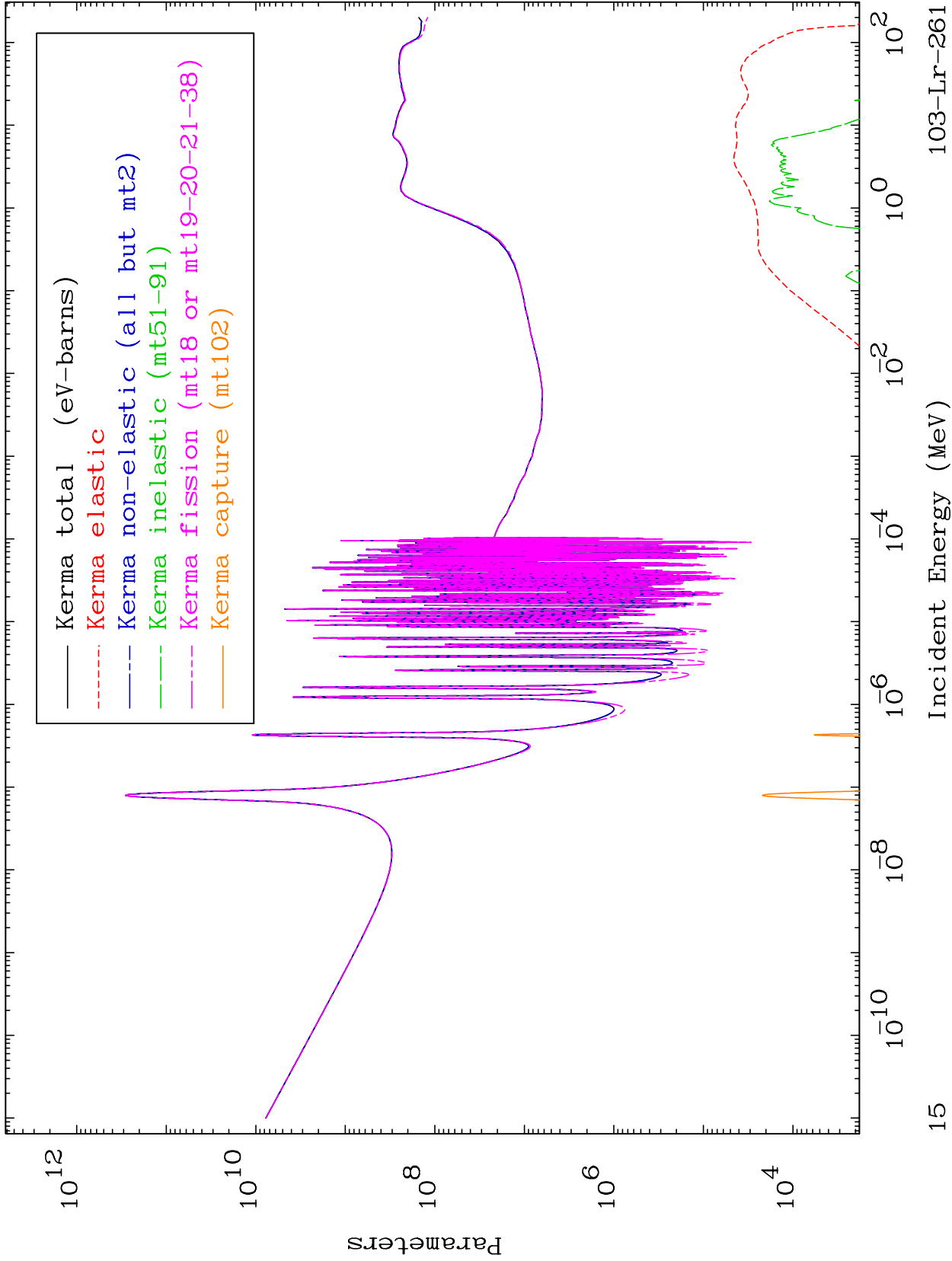
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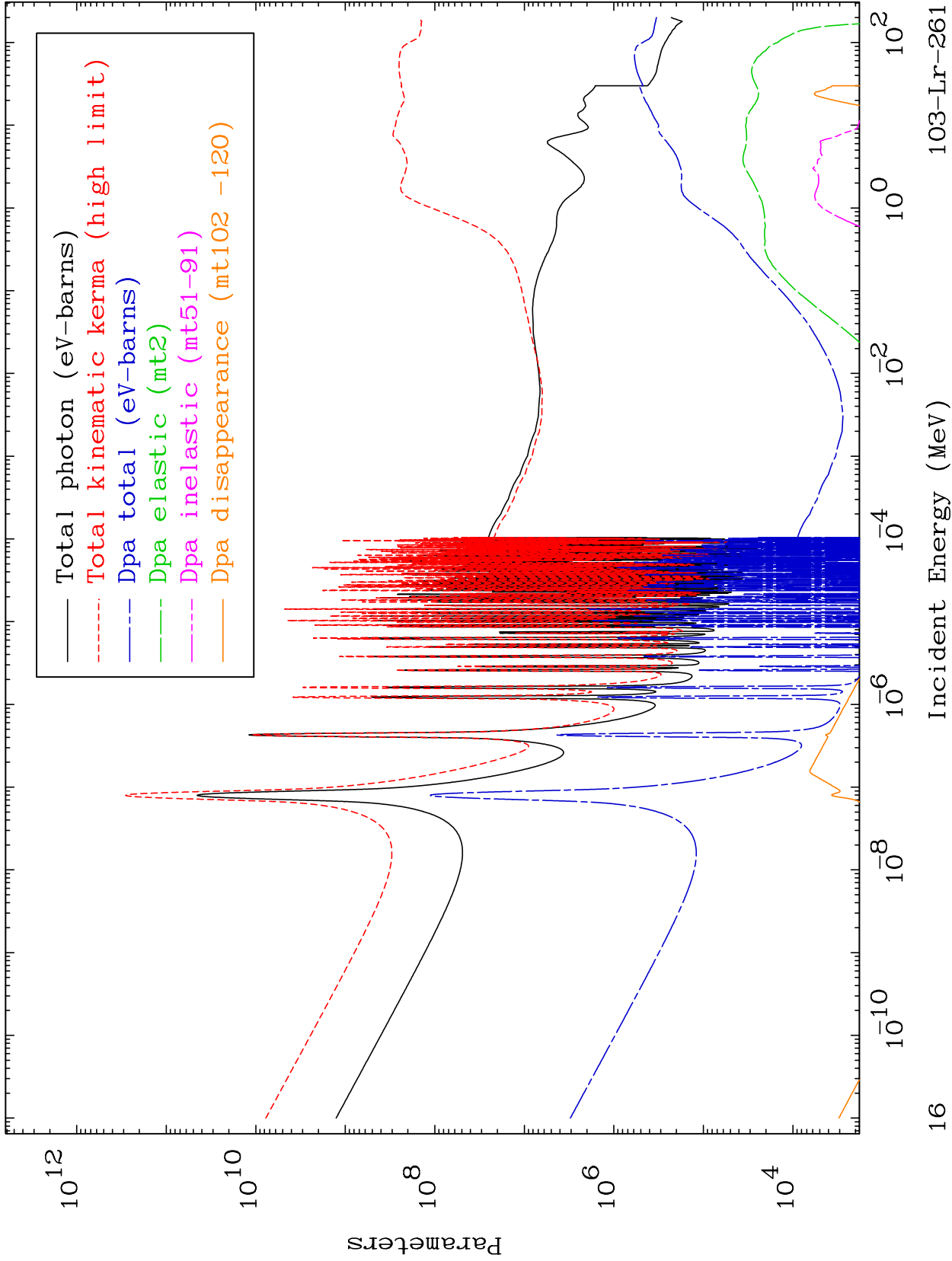
MAT 9990

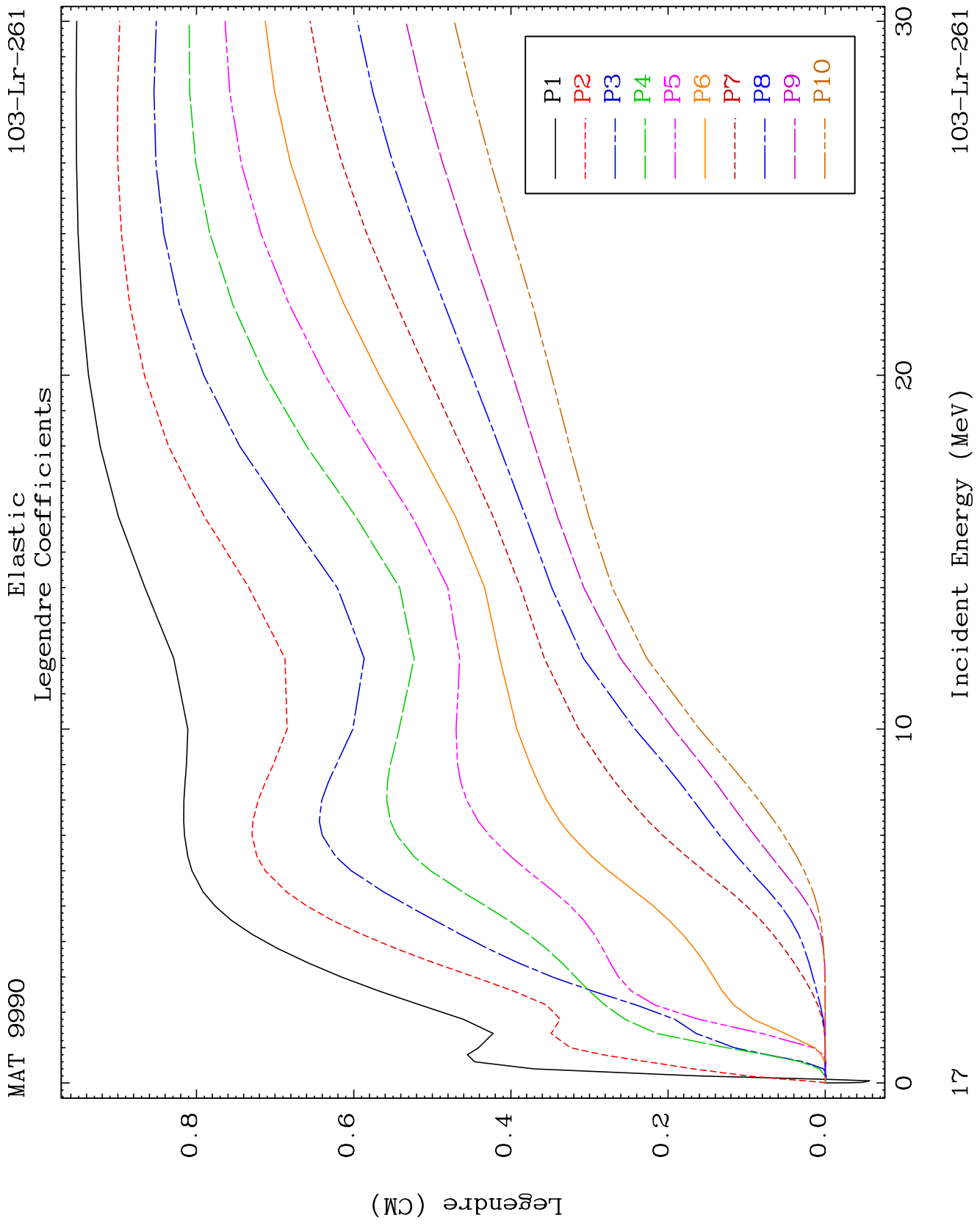
(n,α) Levels
293 Kelvin Cross Sections

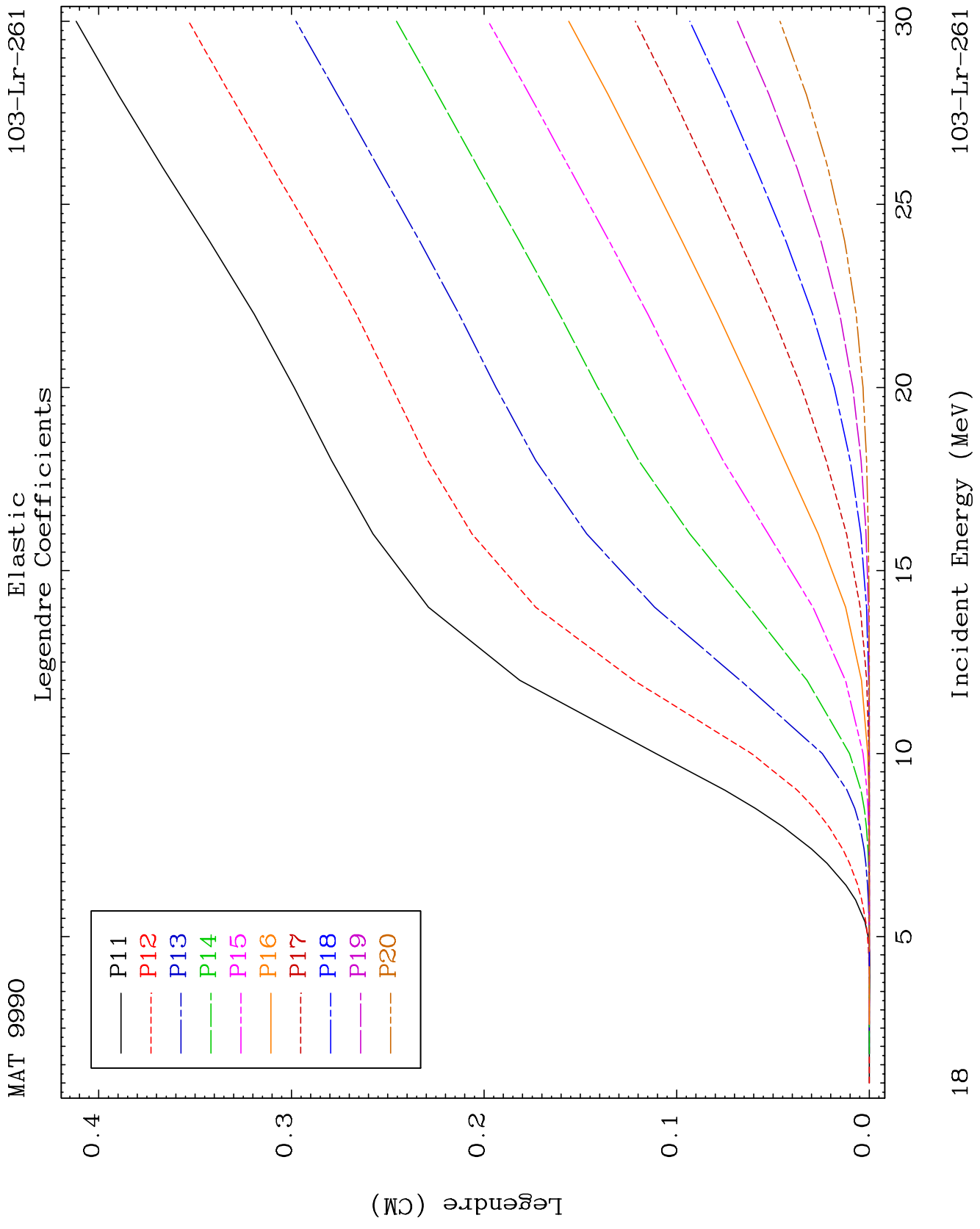
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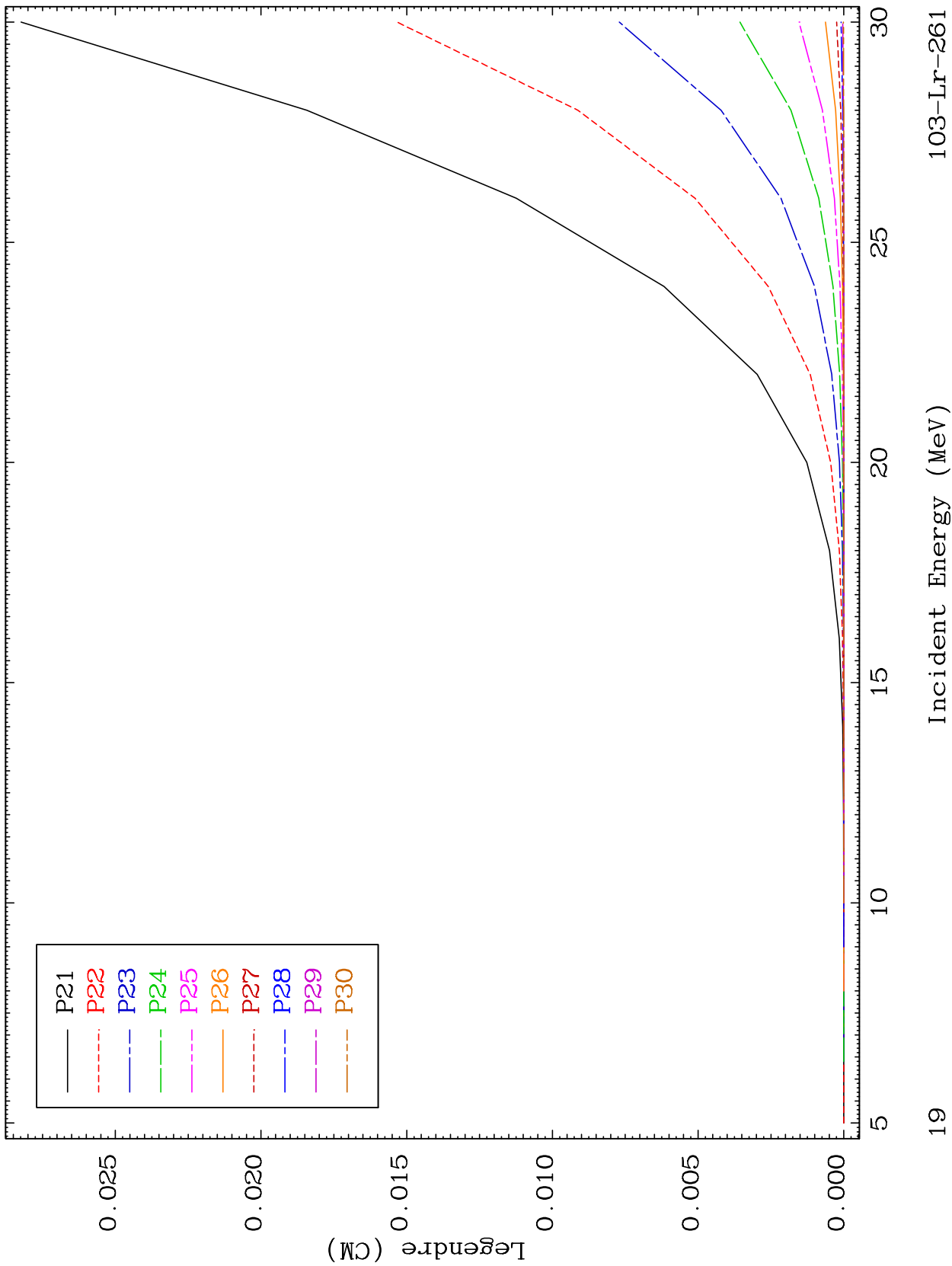




MAT 9990

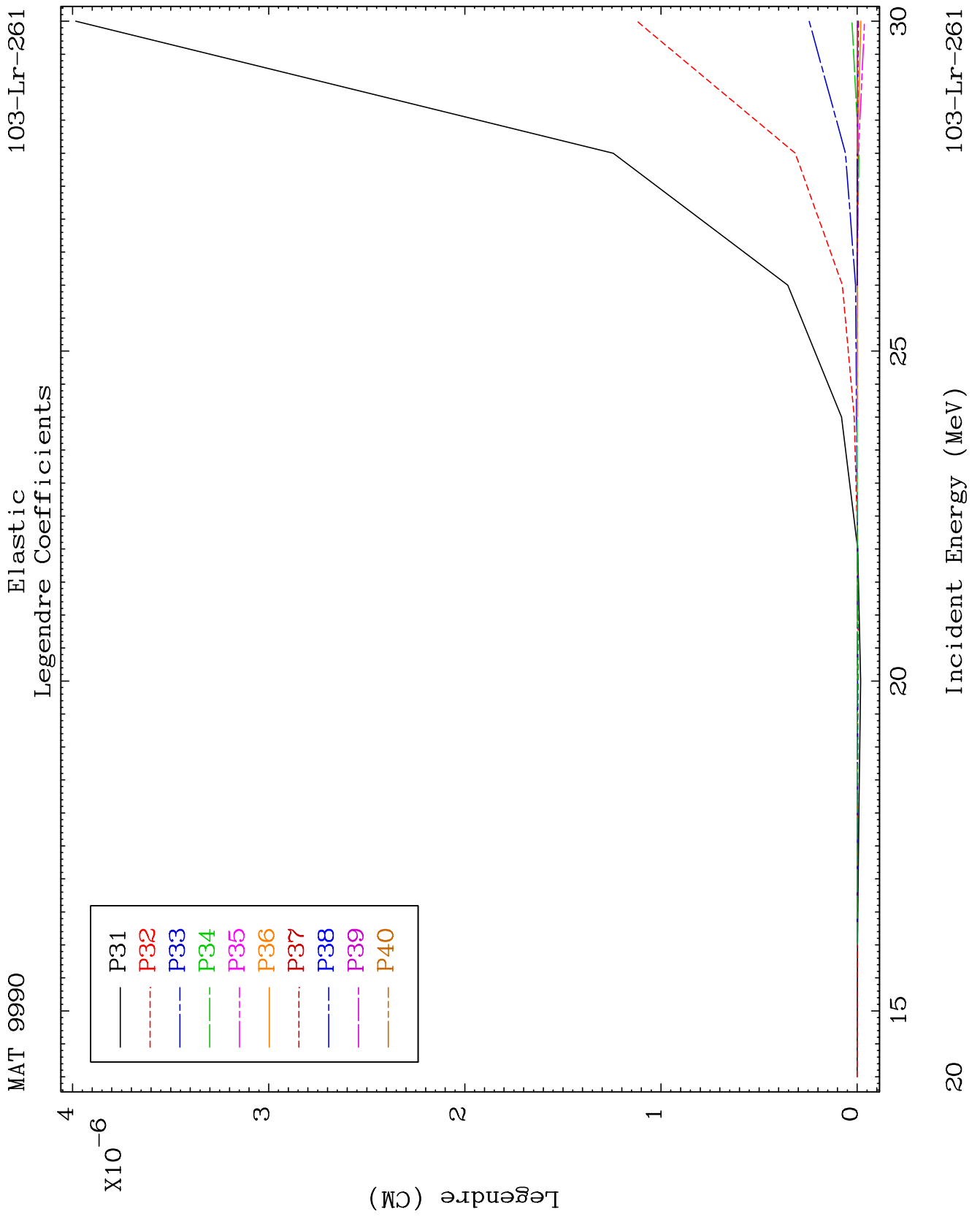
Elastic Legendre Coefficients

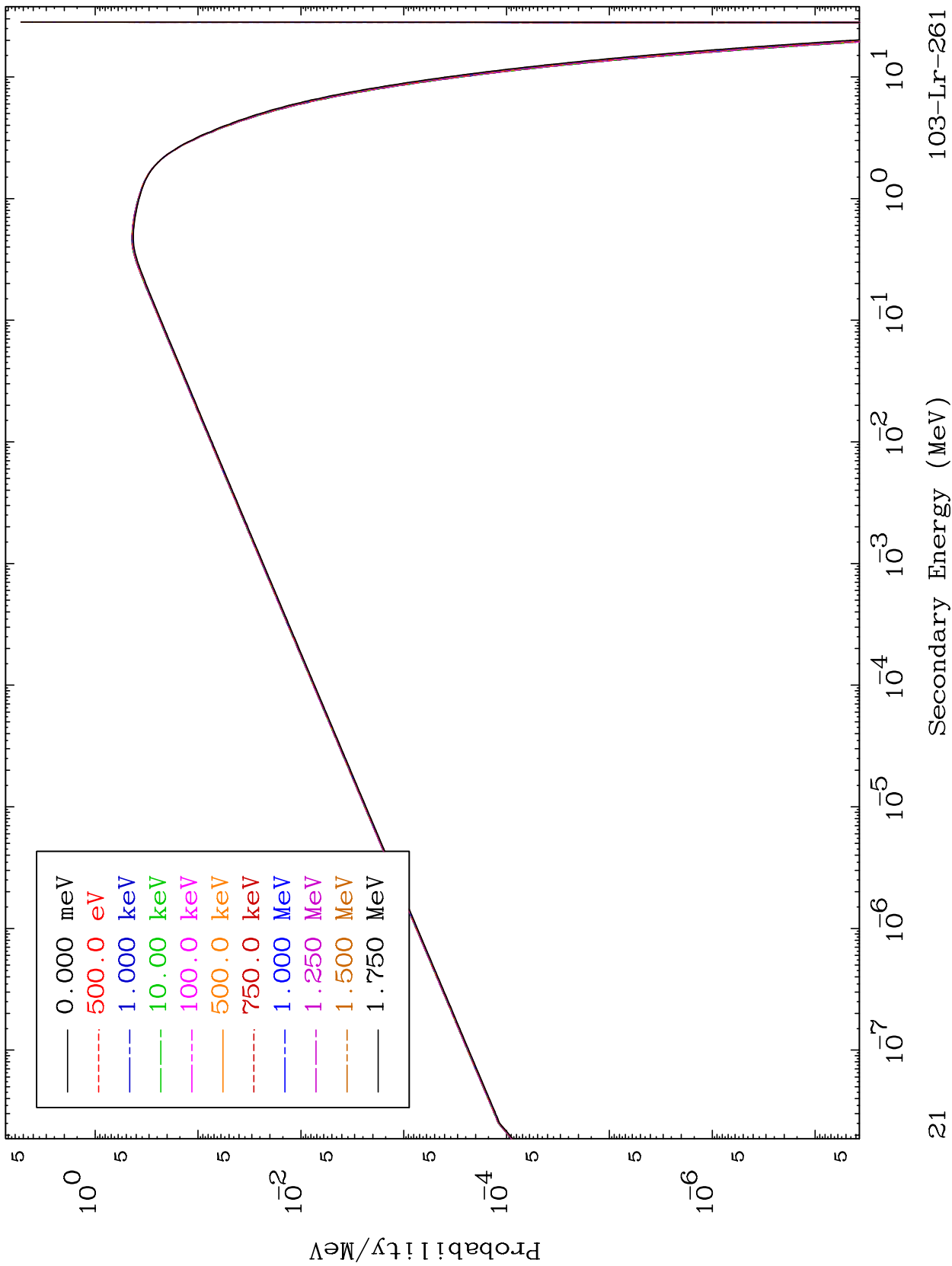
103-Lr-261



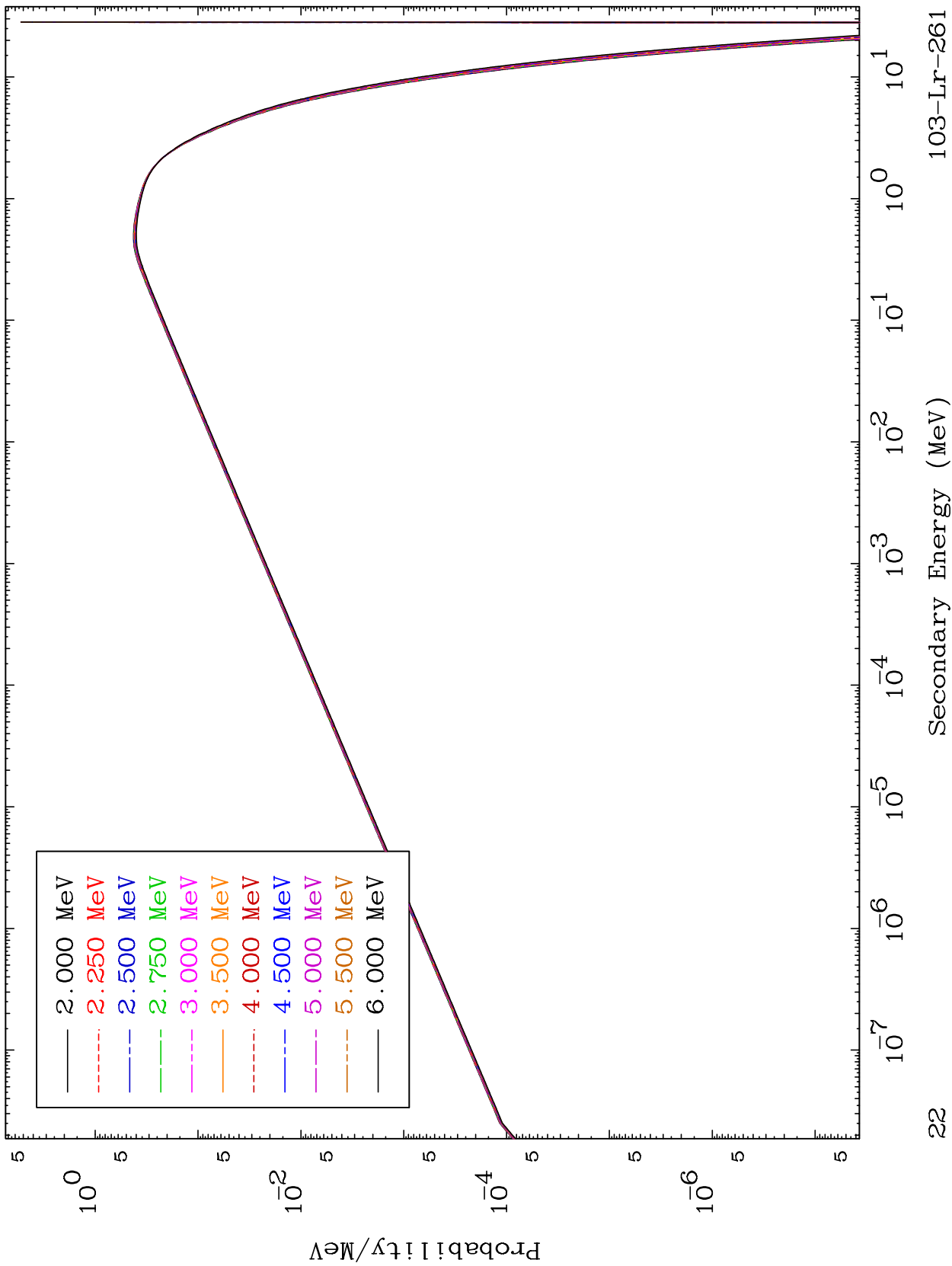
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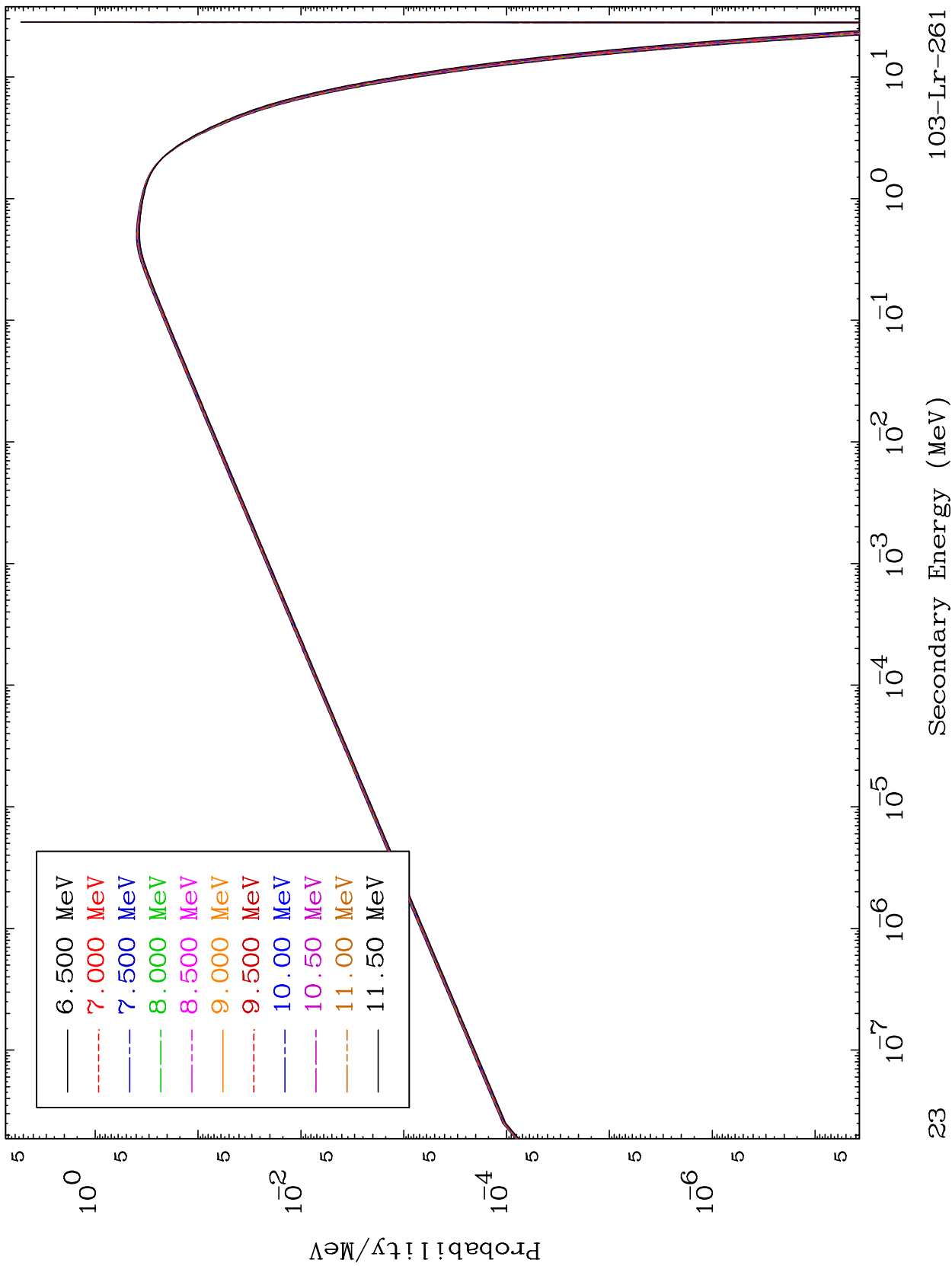
103-Lr-261



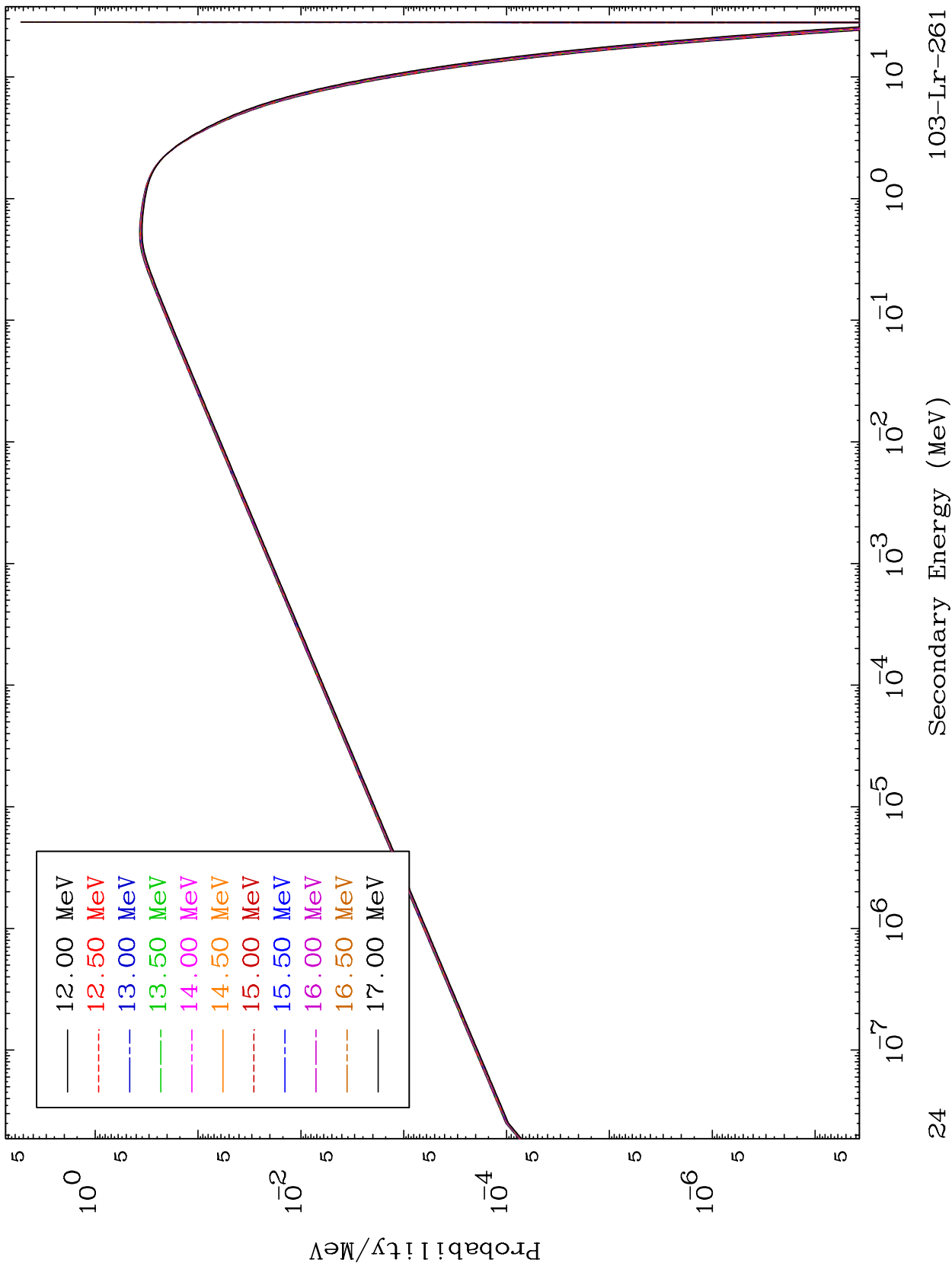


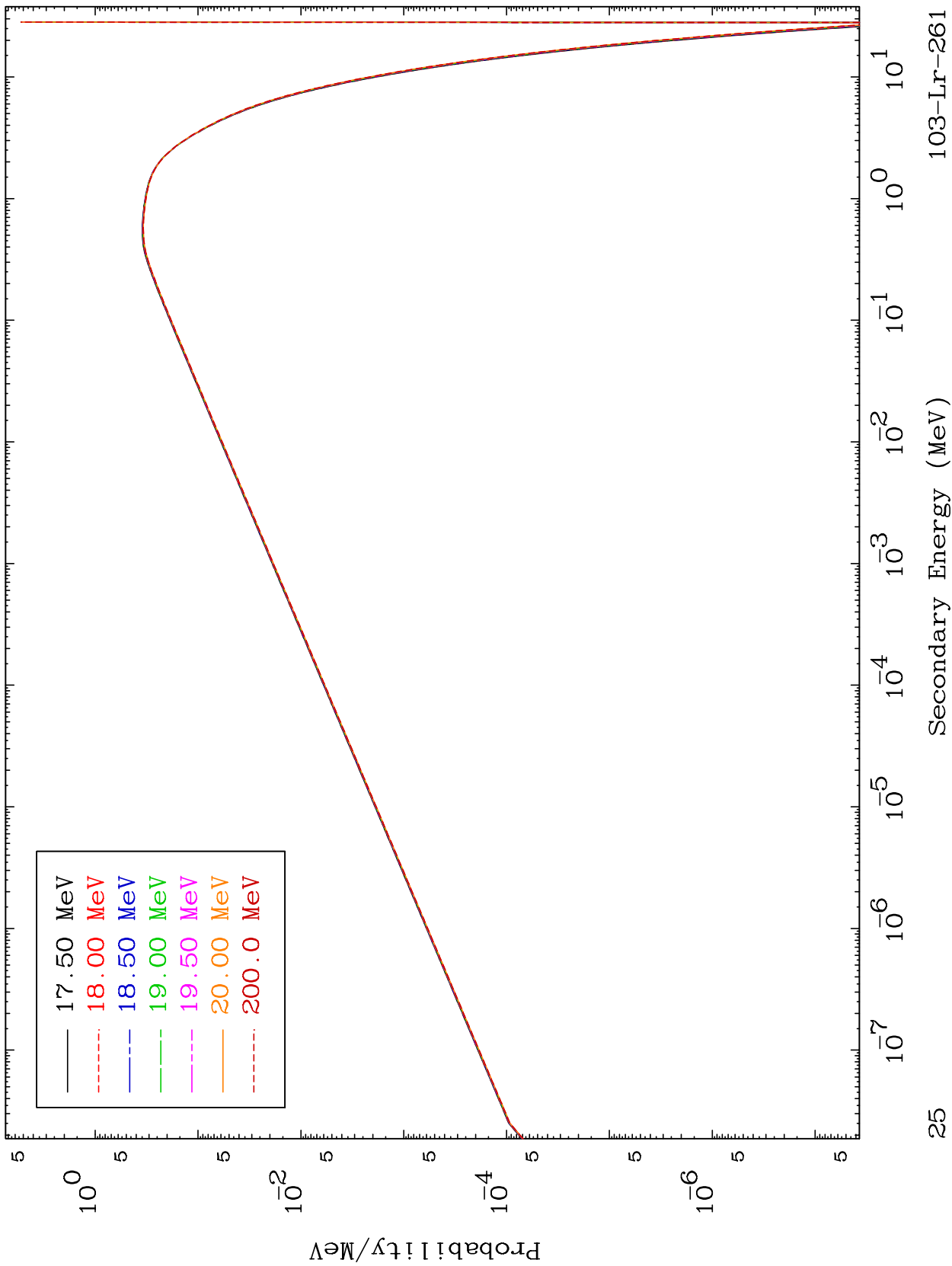
Fission
Energy Distribution





Fission
Energy Distribution



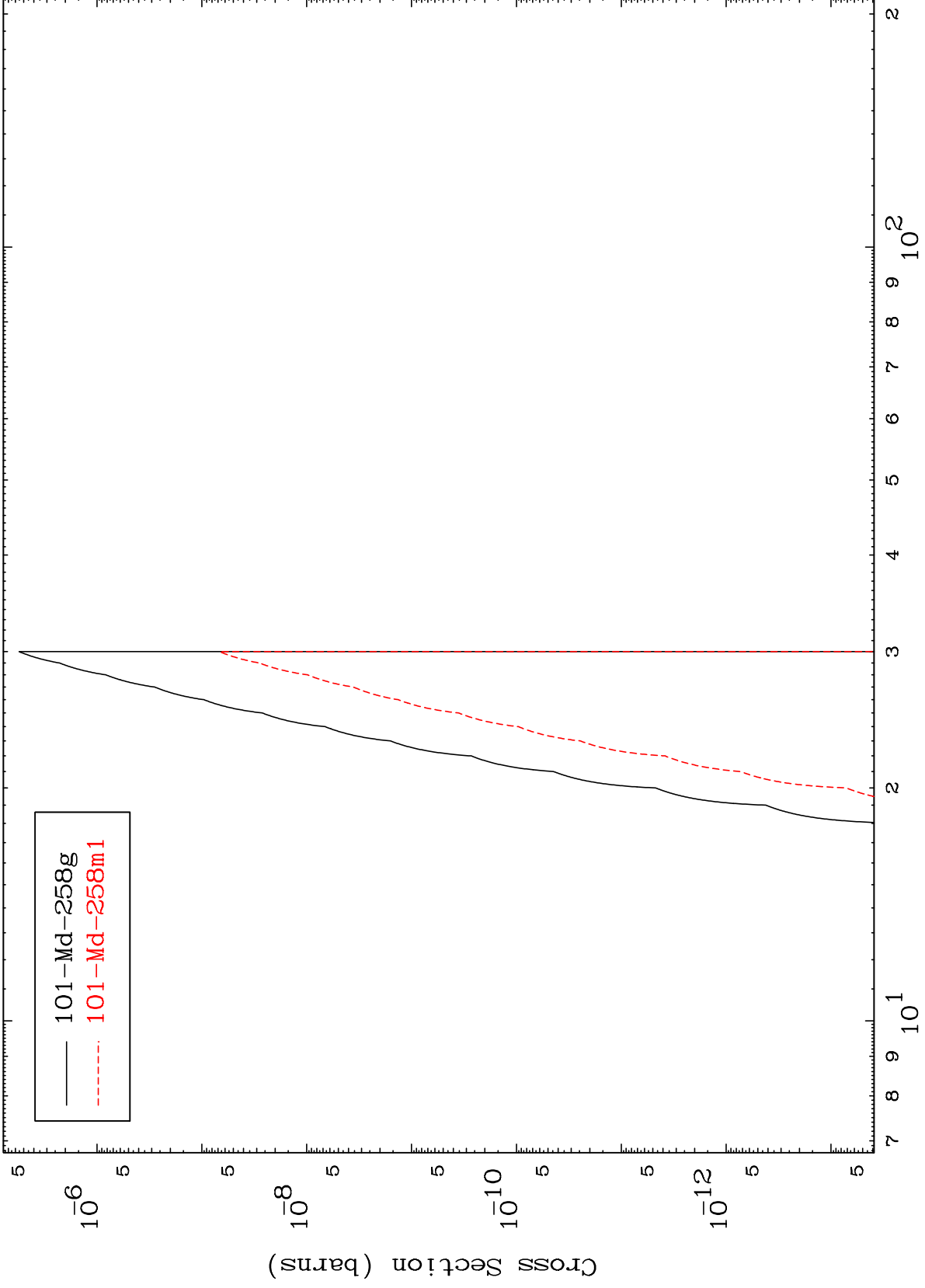


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(n,n') He-3

103-Lr-261

Radionuclide Production Cross Section



26

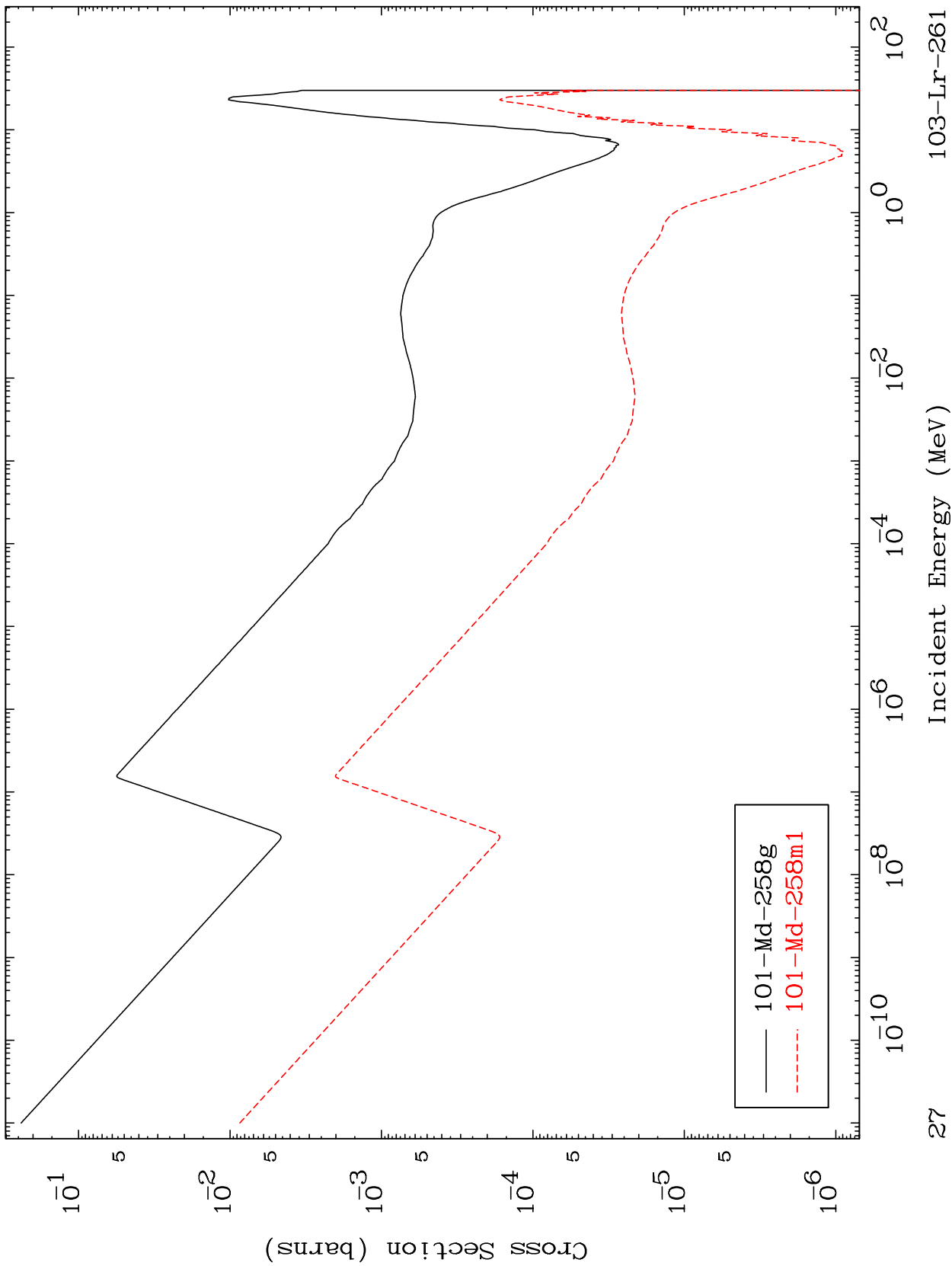
Incident Energy (MeV)

103-Lr-261

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103-Lr-261

Radionuclide Production Cross Section
(n, α)



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(n,2α)

103-Lr-261

Radionuclide Production Cross Section

