

Program EVALPLOT  
(Version 2015-2)

by

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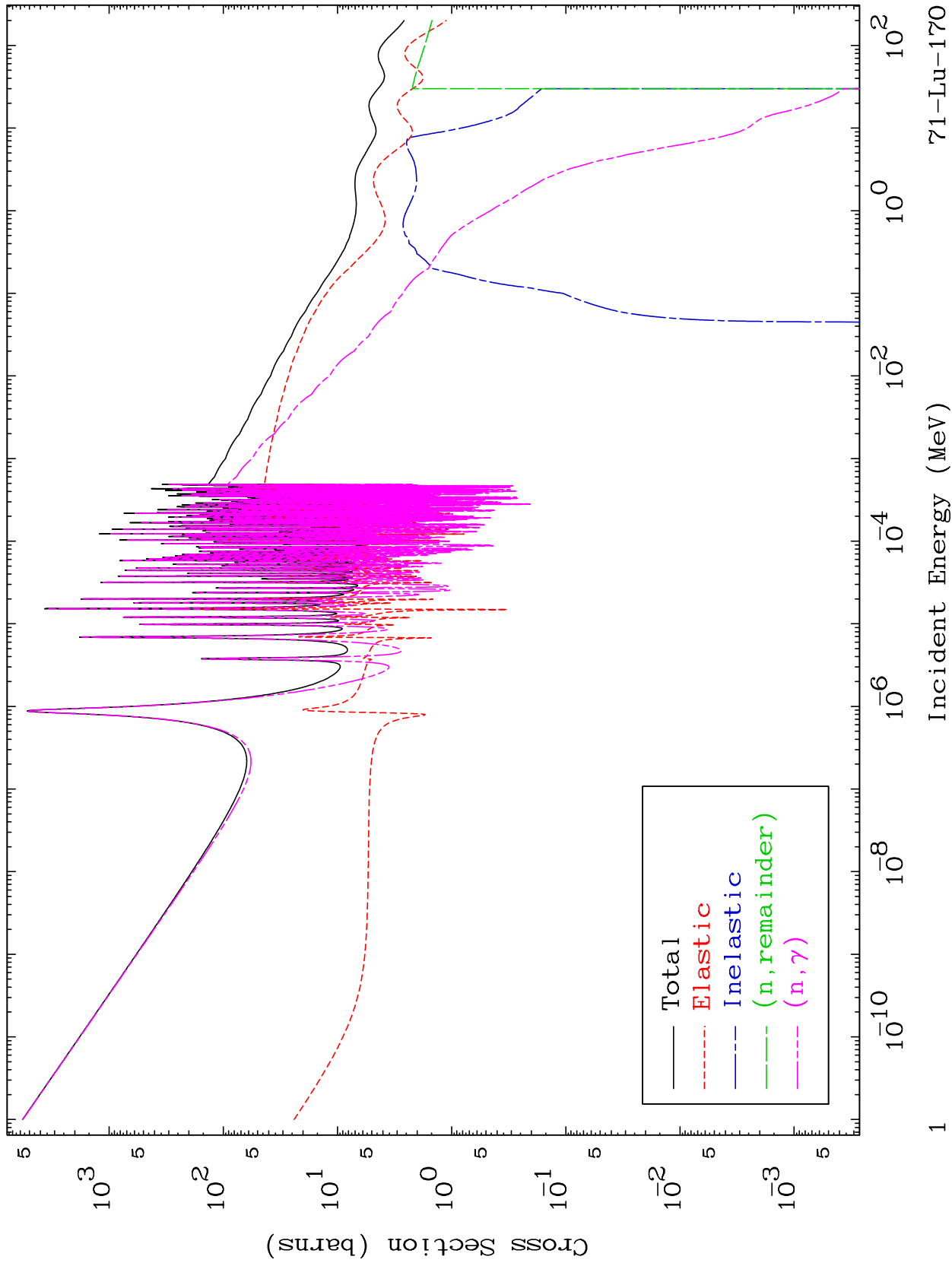
Press Mouse Button to Start

MAT 7110

Major

293 Kelvin Cross Sections

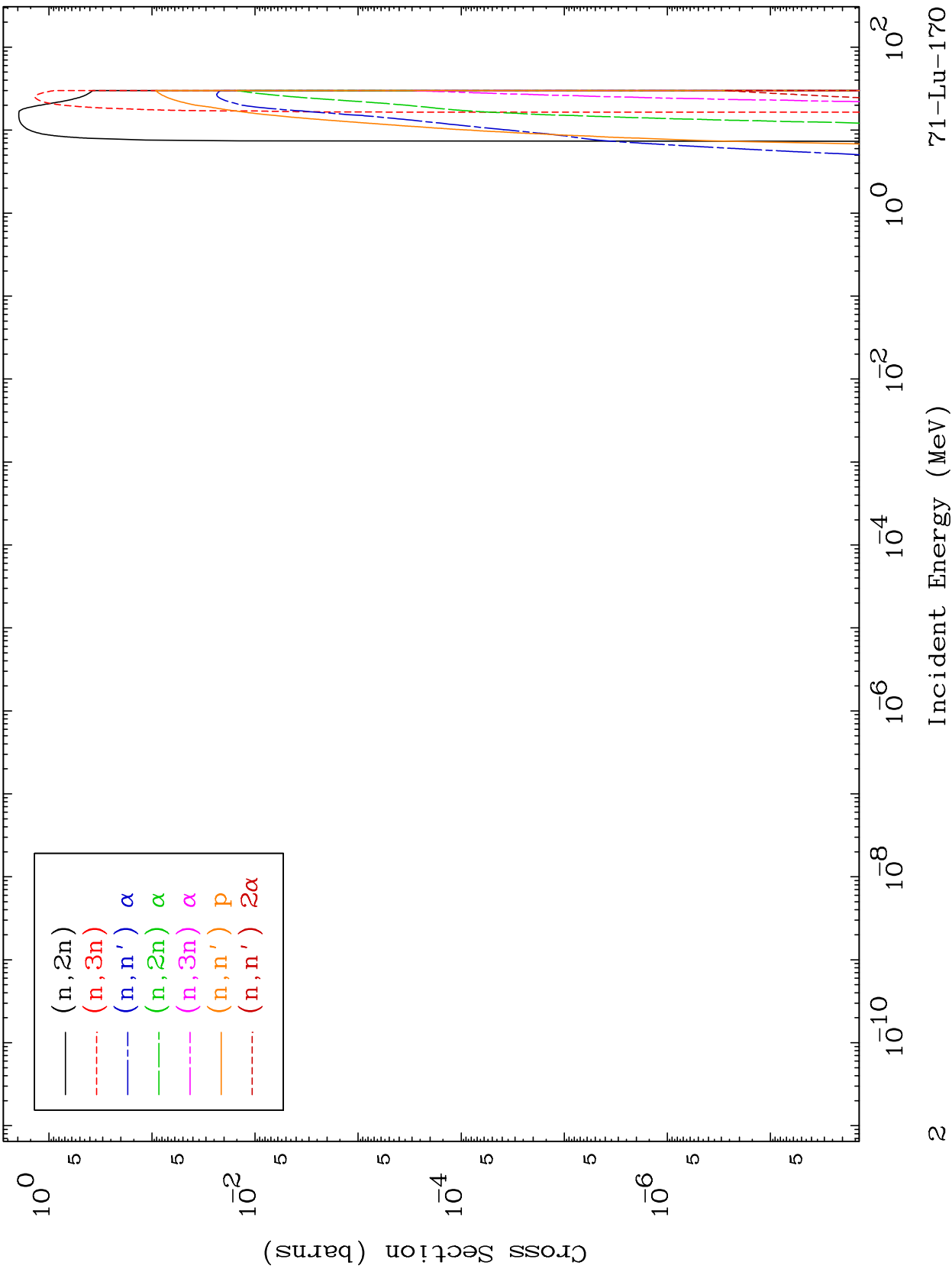
71-Lu-170



MAT 7110

Neutron Production  
293 Kelvin Cross Sections

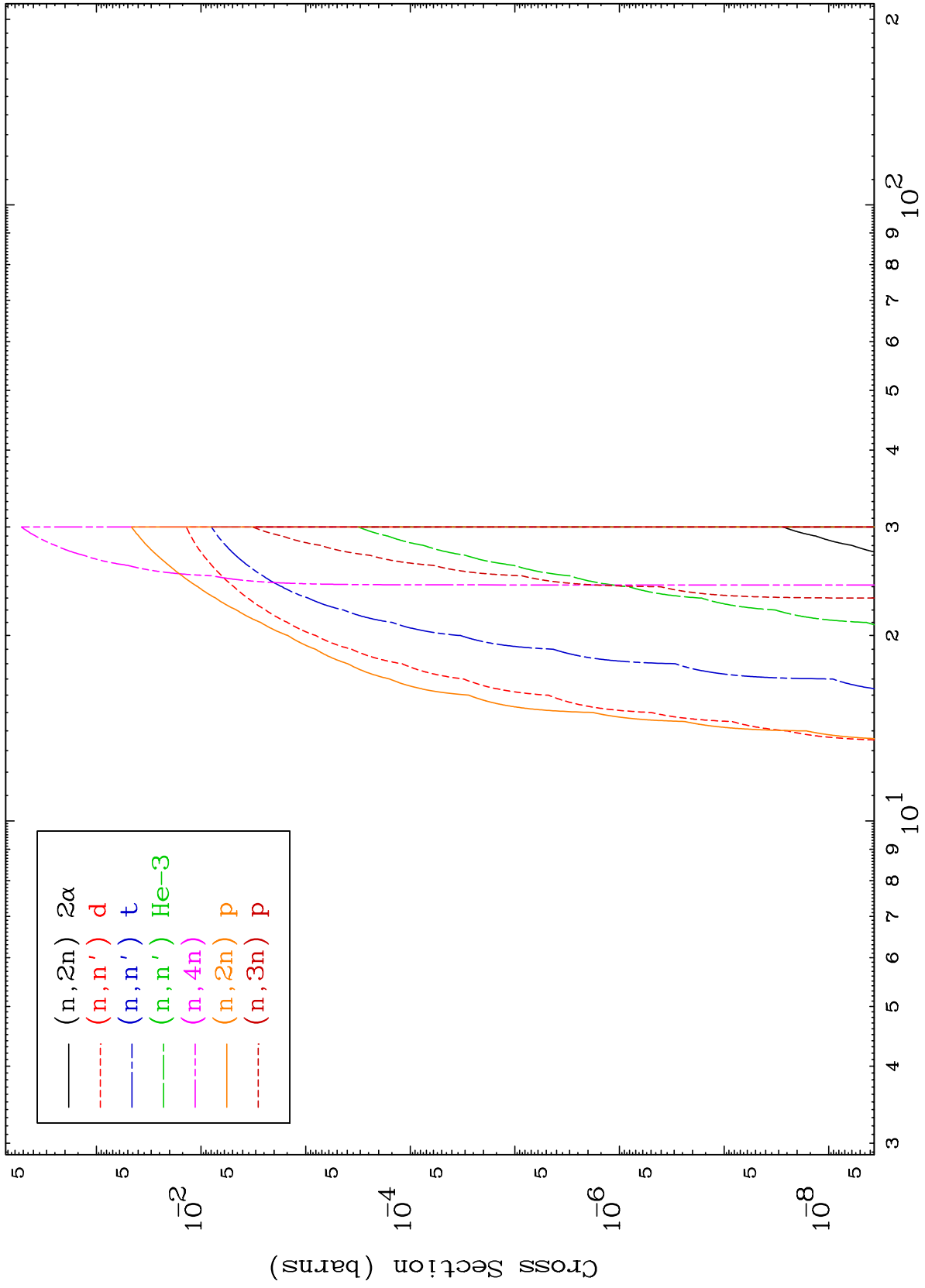
71-Lu-170



MAT 7110

Neutron Production  
293 Kelvin Cross Sections

71-Lu-170



Incident Energy (MeV)

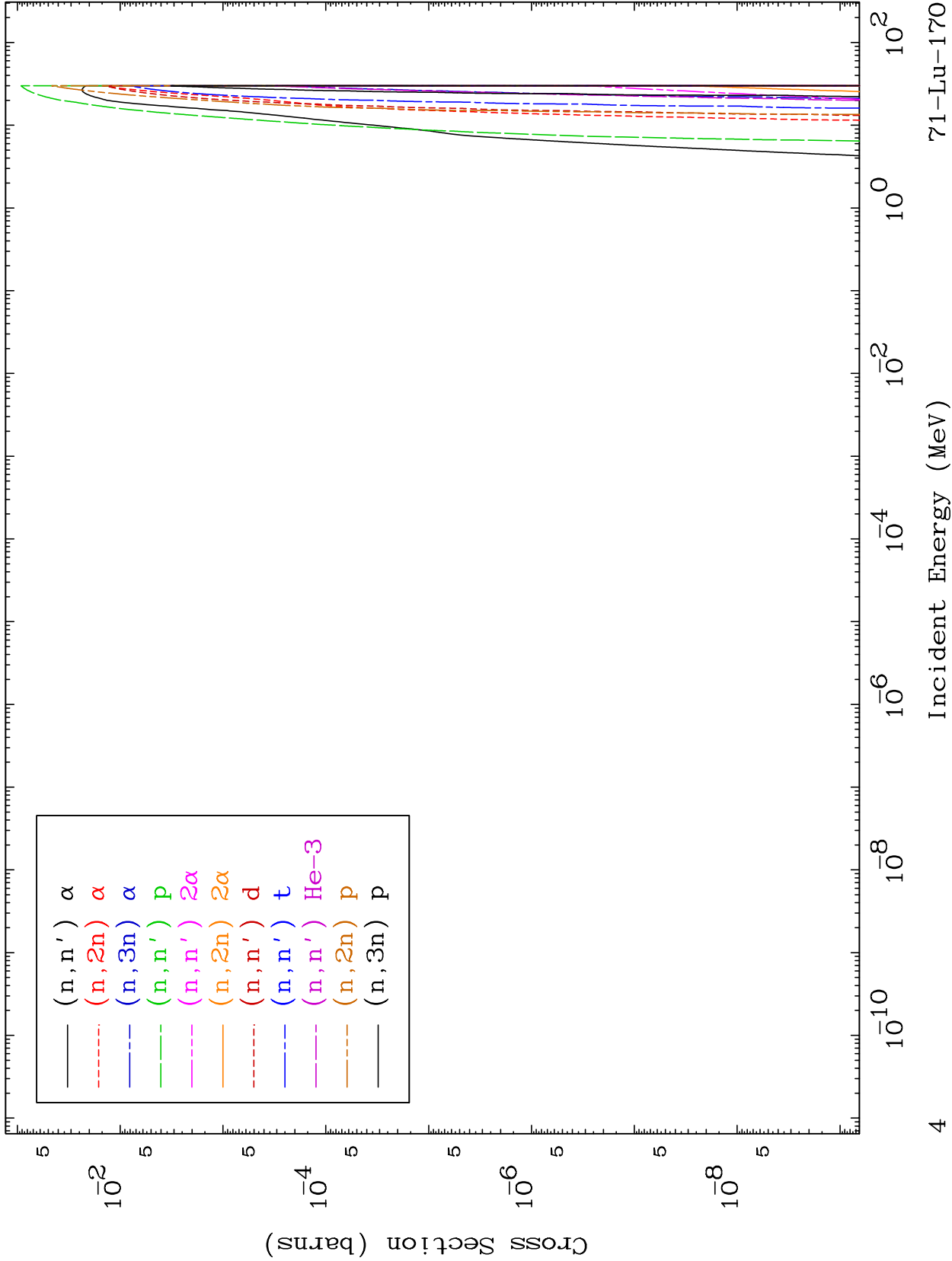
71-Lu-170

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MAT 7110

Charged Particle  
293 Kelvin Cross Sections

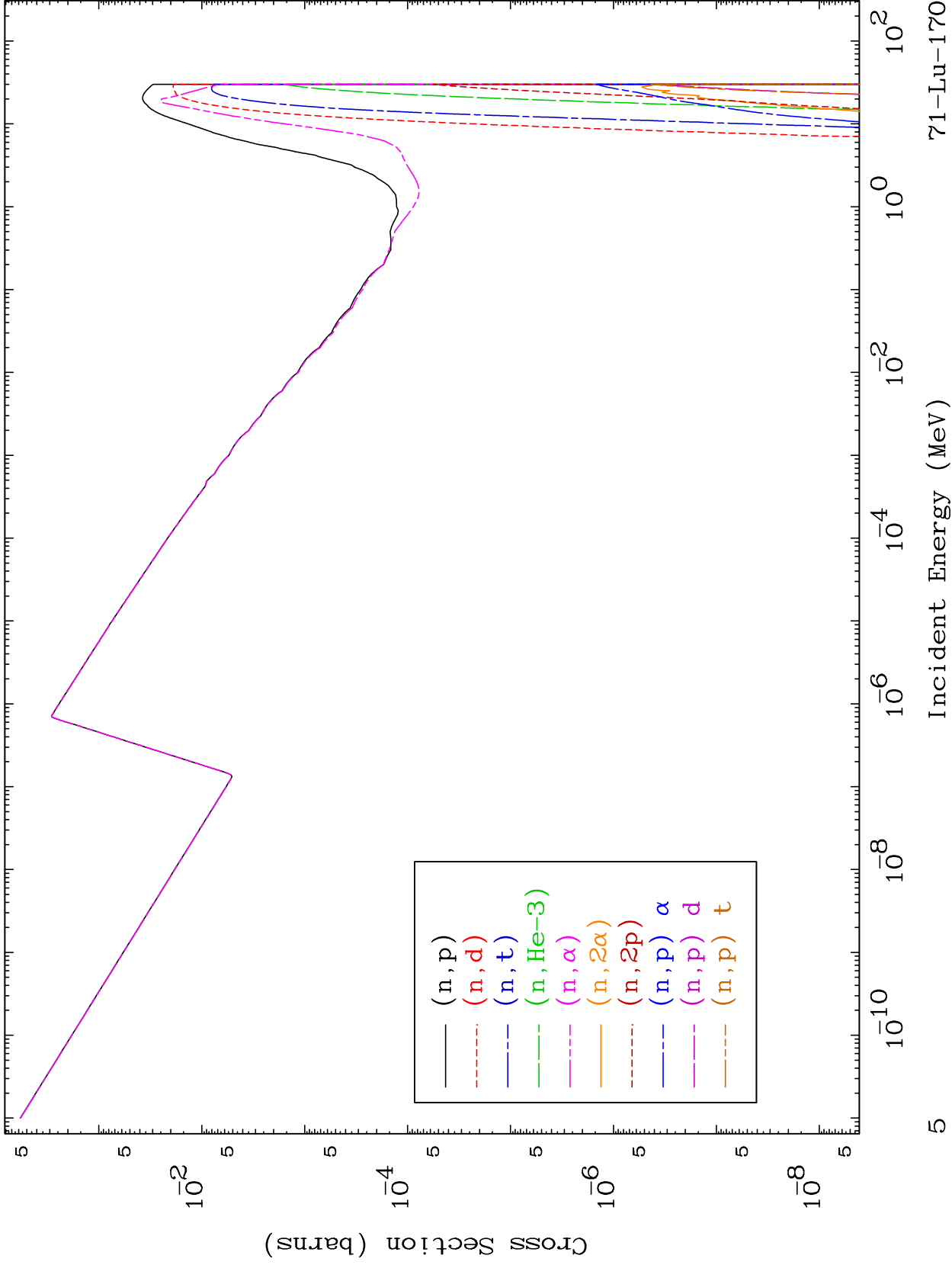
71-Lu-170



MAT 7110

293 Kelvin Cross Sections

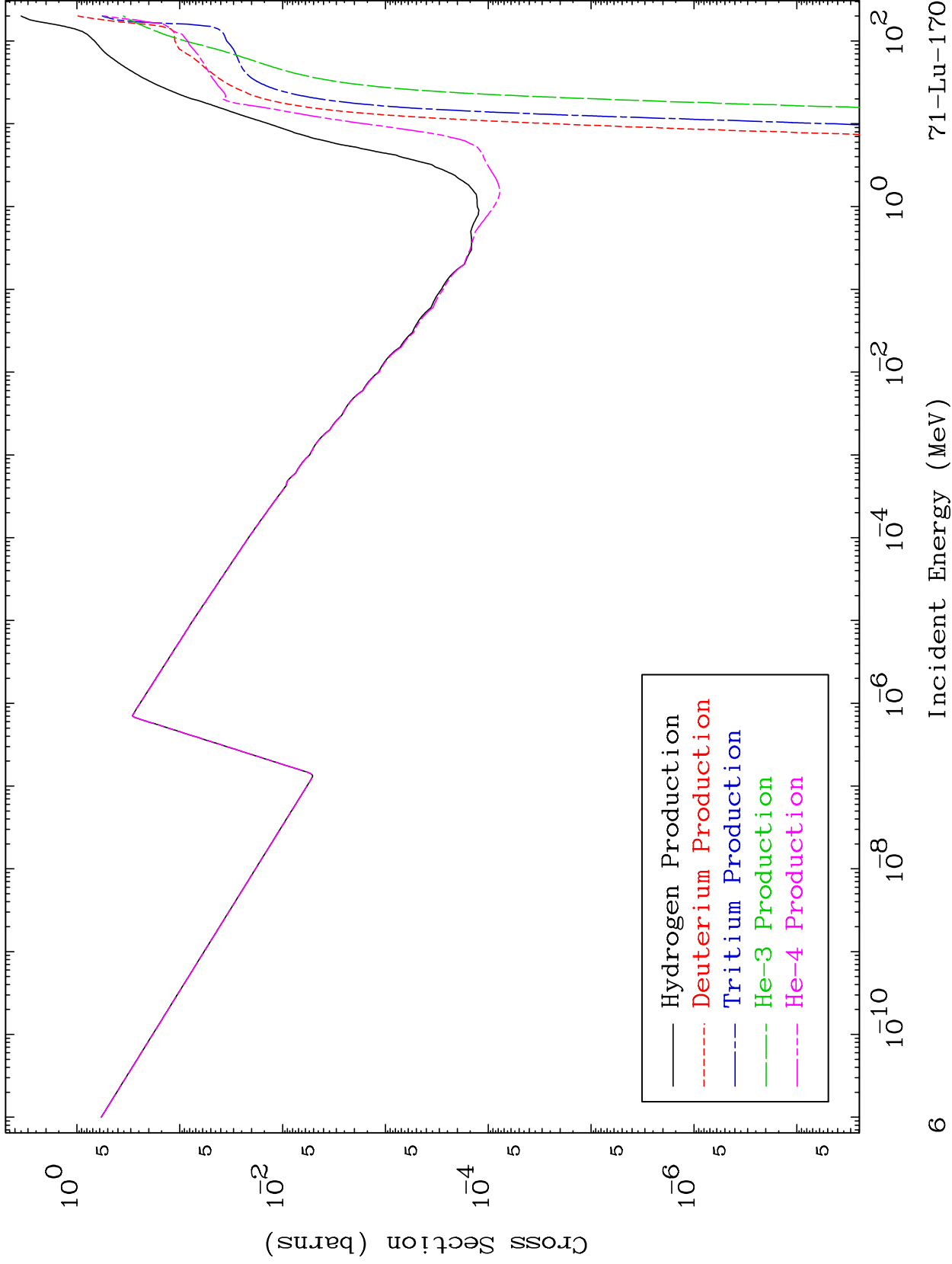
71-Lu-170



MAT 7110

Particle Production  
293 Kelvin Cross Sections

71-Lu-170

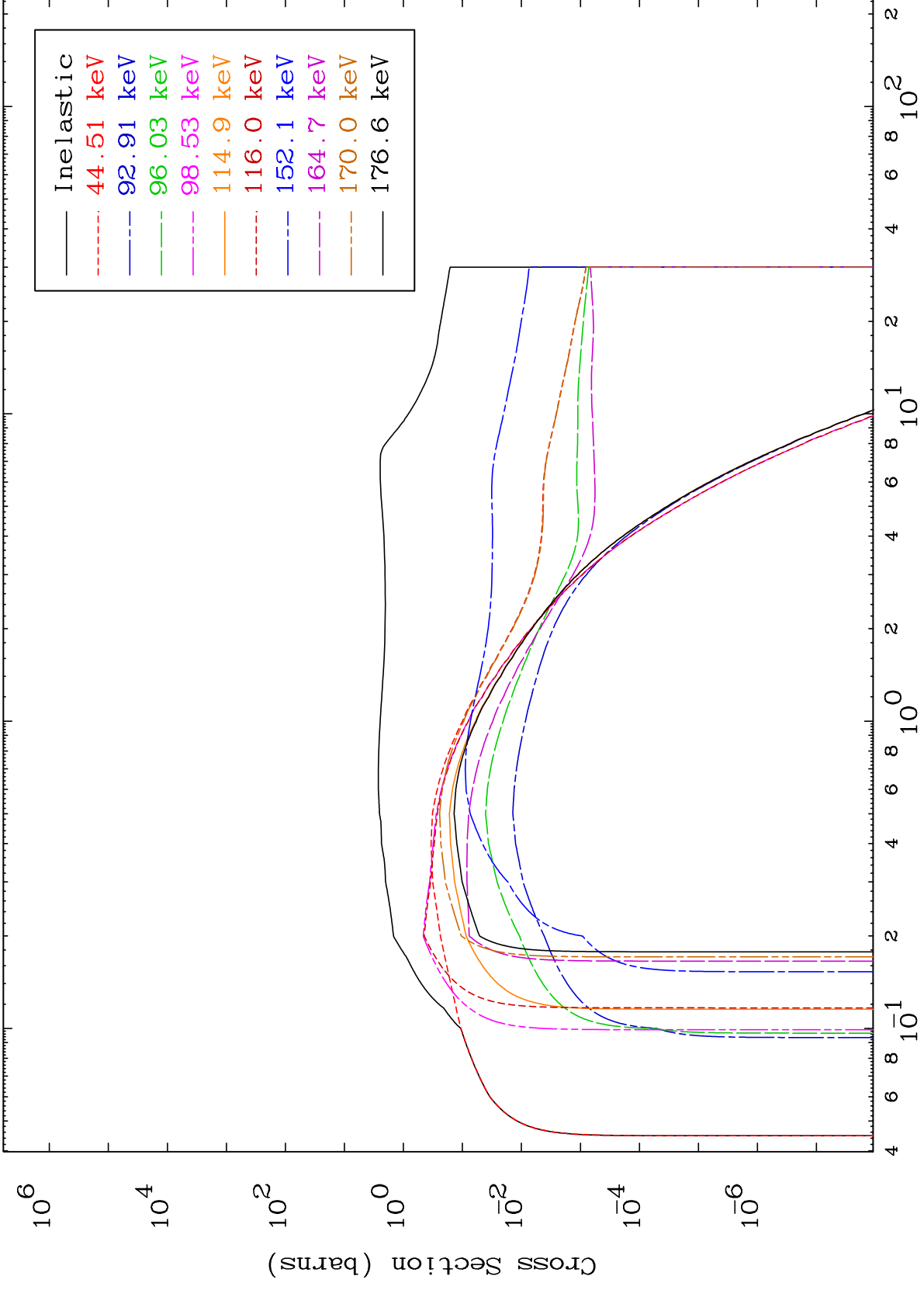


MAT 7110

(n,n') Level

71-Lu-170

293 Kelvin Cross Sections

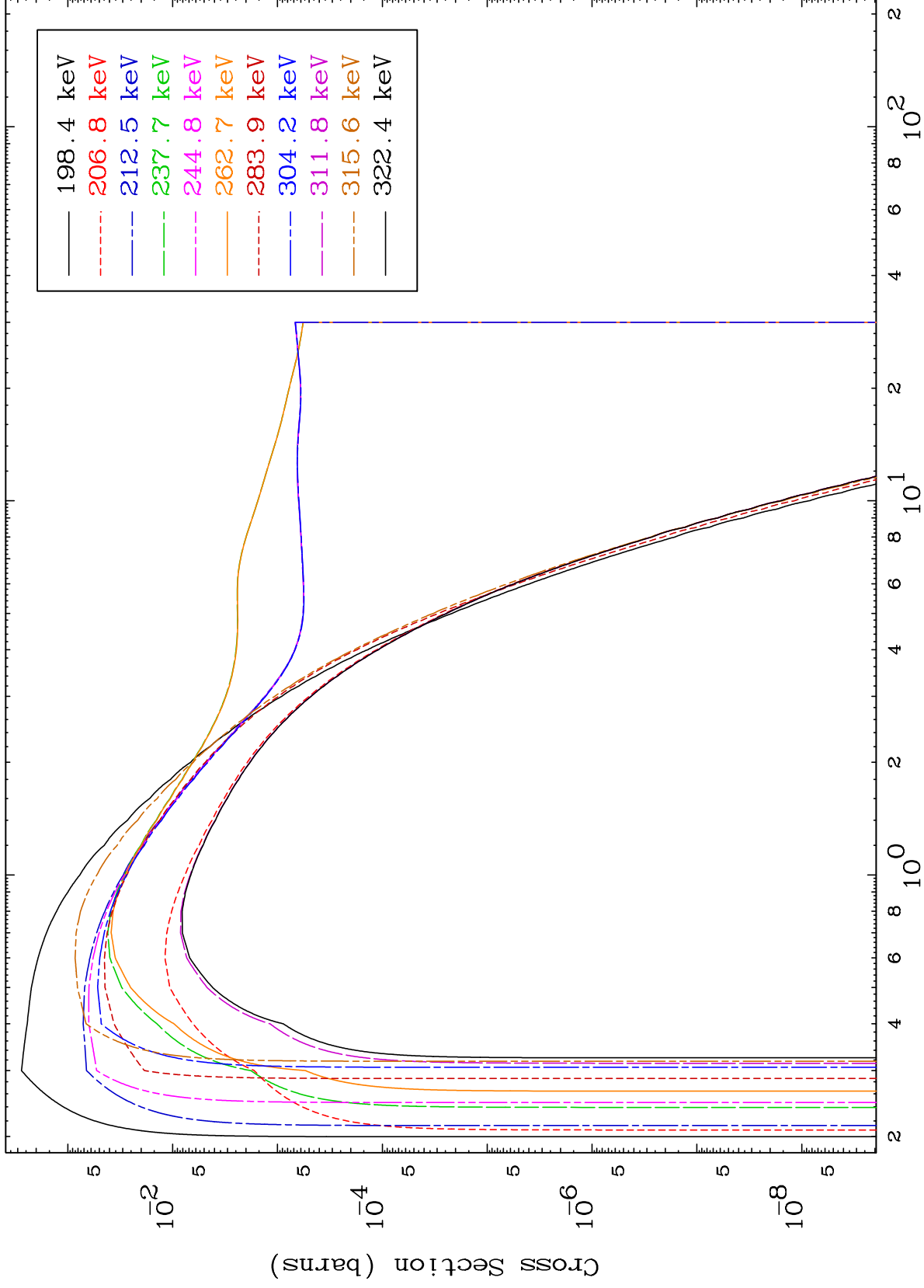




MAT 7110

(n,n') Level  
293 Kelvin Cross Sections

71-Lu-170



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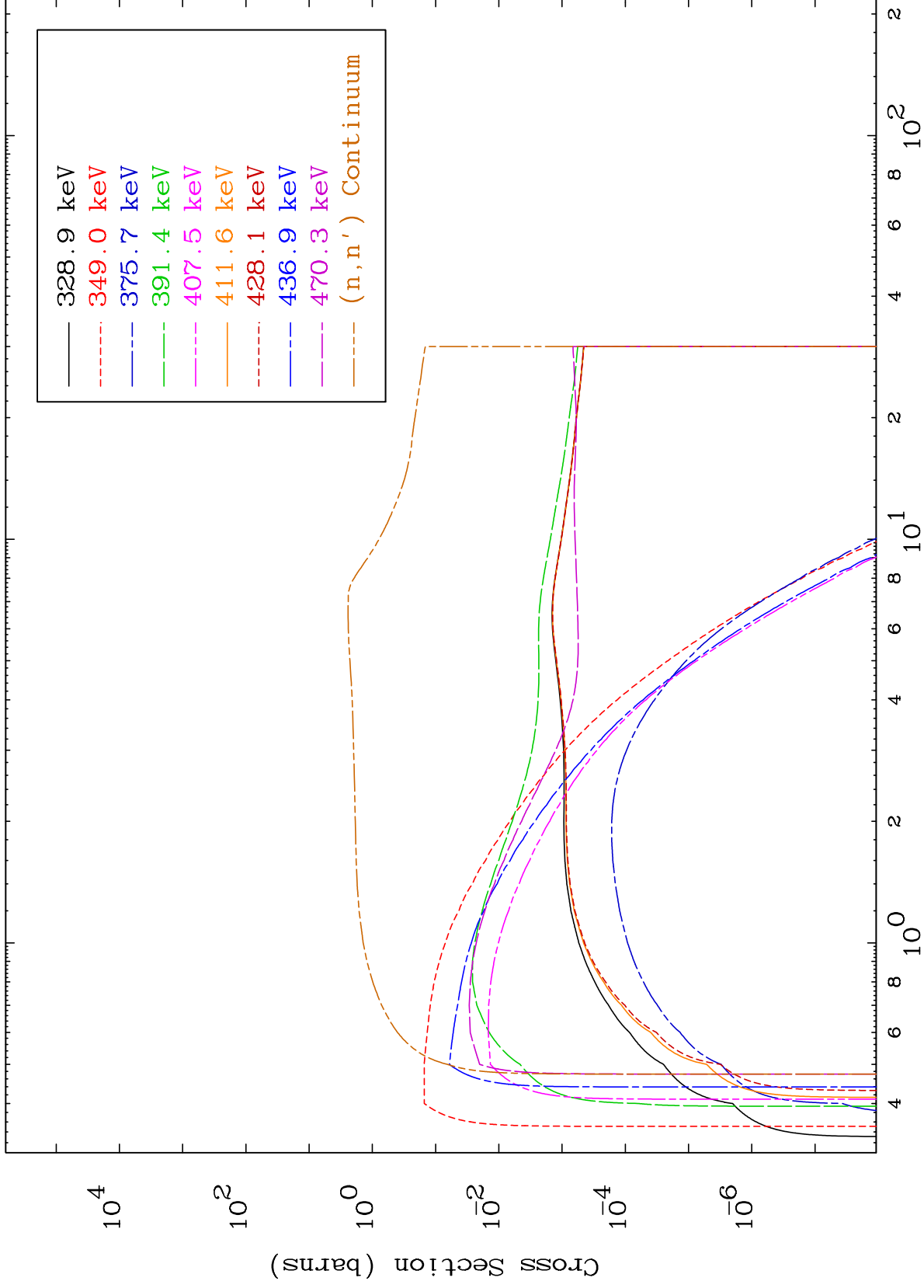
Incident Energy (MeV)

71-Lu-170

MAT 7110

(n,n') Level  
293 Kelvin Cross Sections

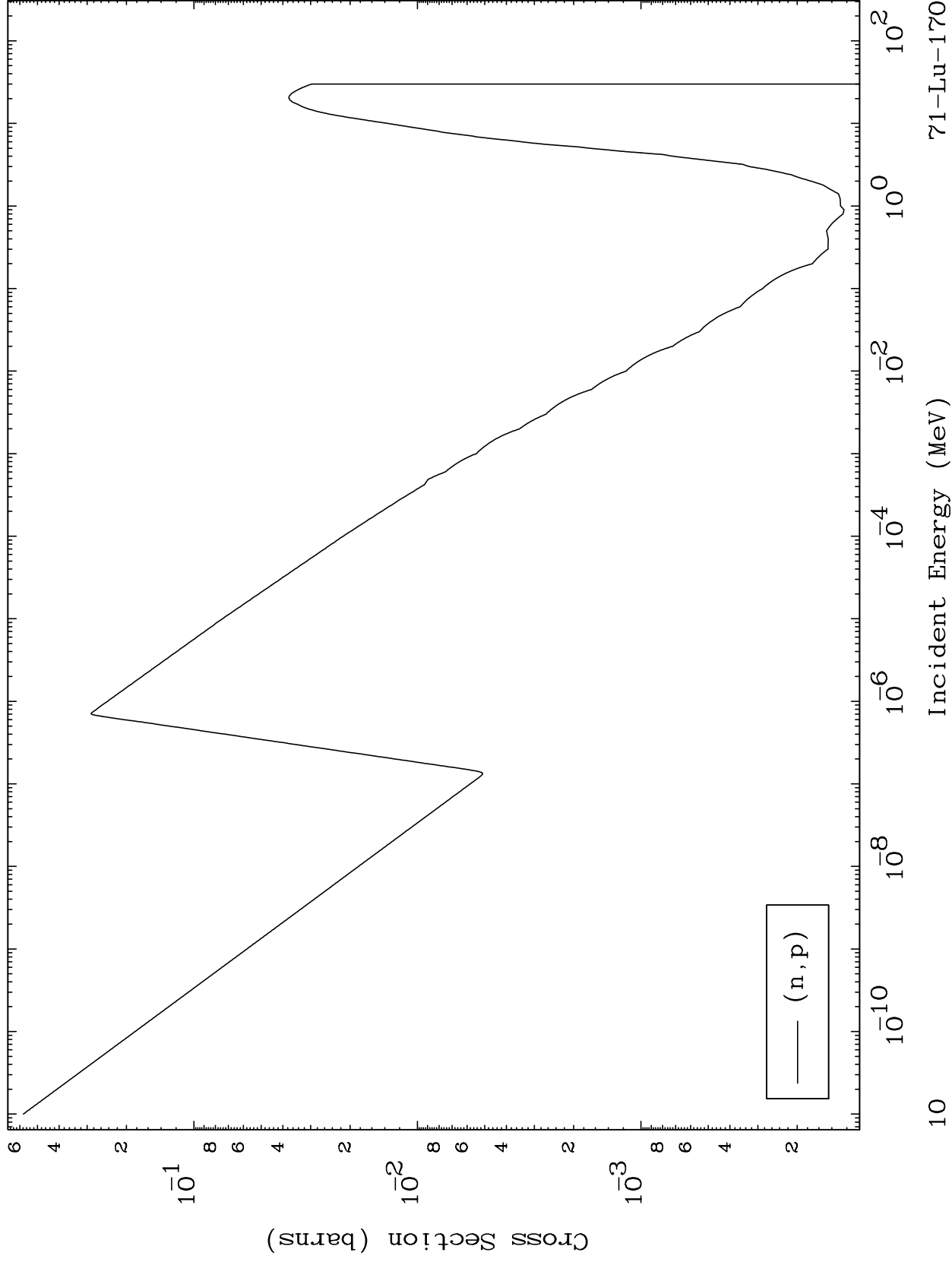
71-Lu-170



MAT 7110

(n,p) Levels  
293 Kelvin Cross Sections

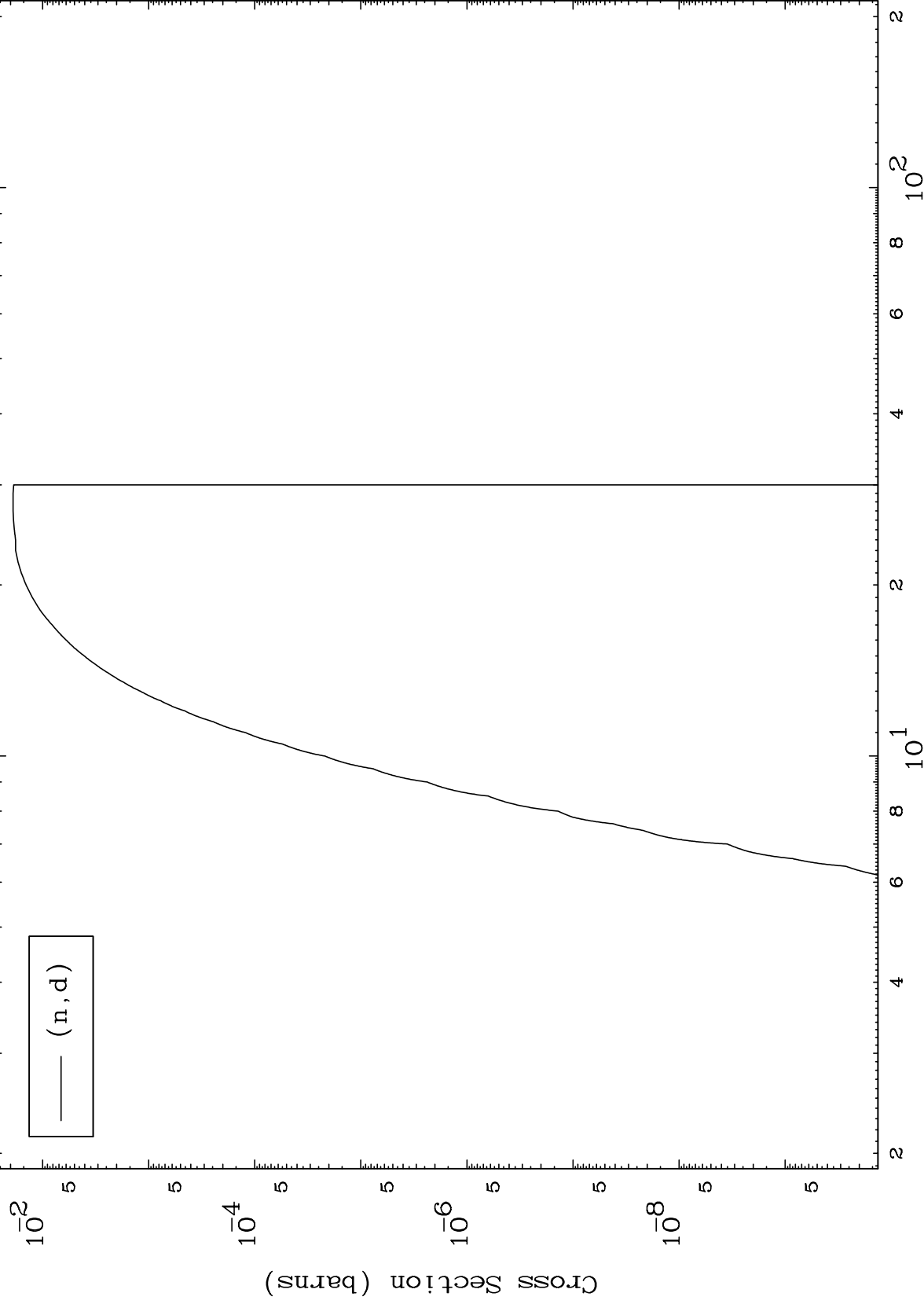
71-Lu-170



MAT 7110

(n,d) Levels  
293 Kelvin Cross Sections

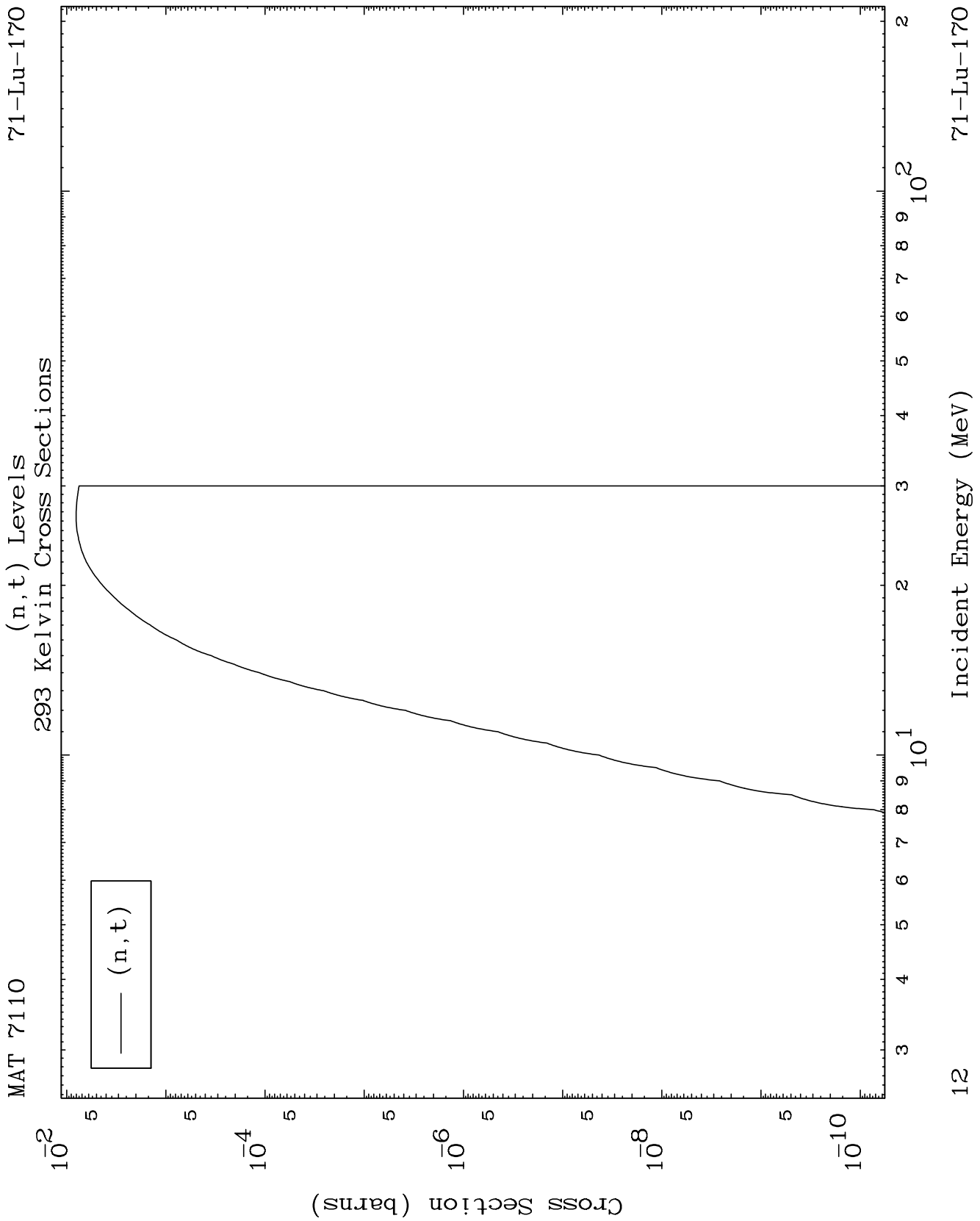
71-Lu-170



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Incident Energy (MeV)

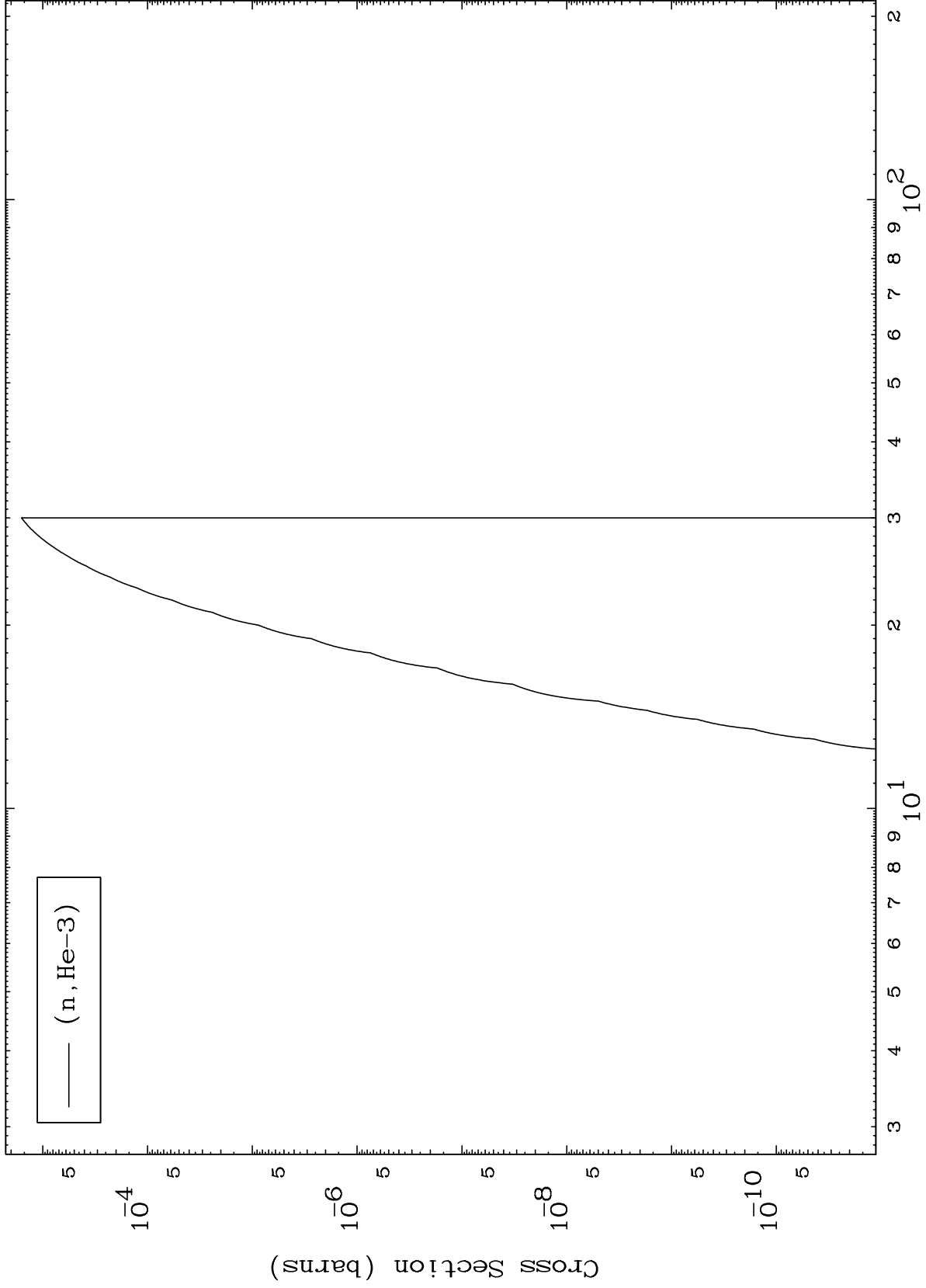
71-Lu-170



MAT 7110

(n,He3) Levels  
293 Kelvin Cross Sections

71-Lu-170



13

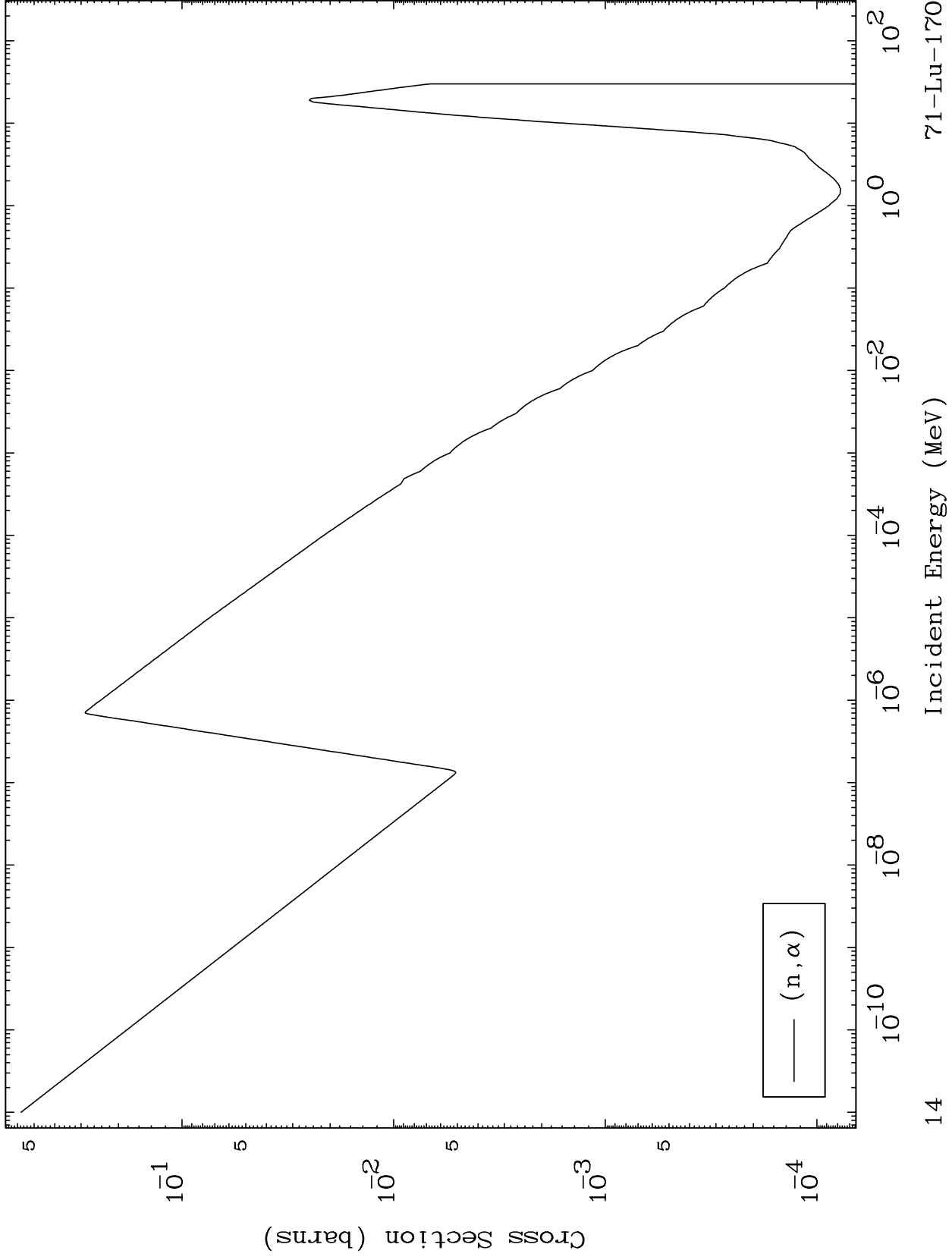
Incident Energy (MeV)

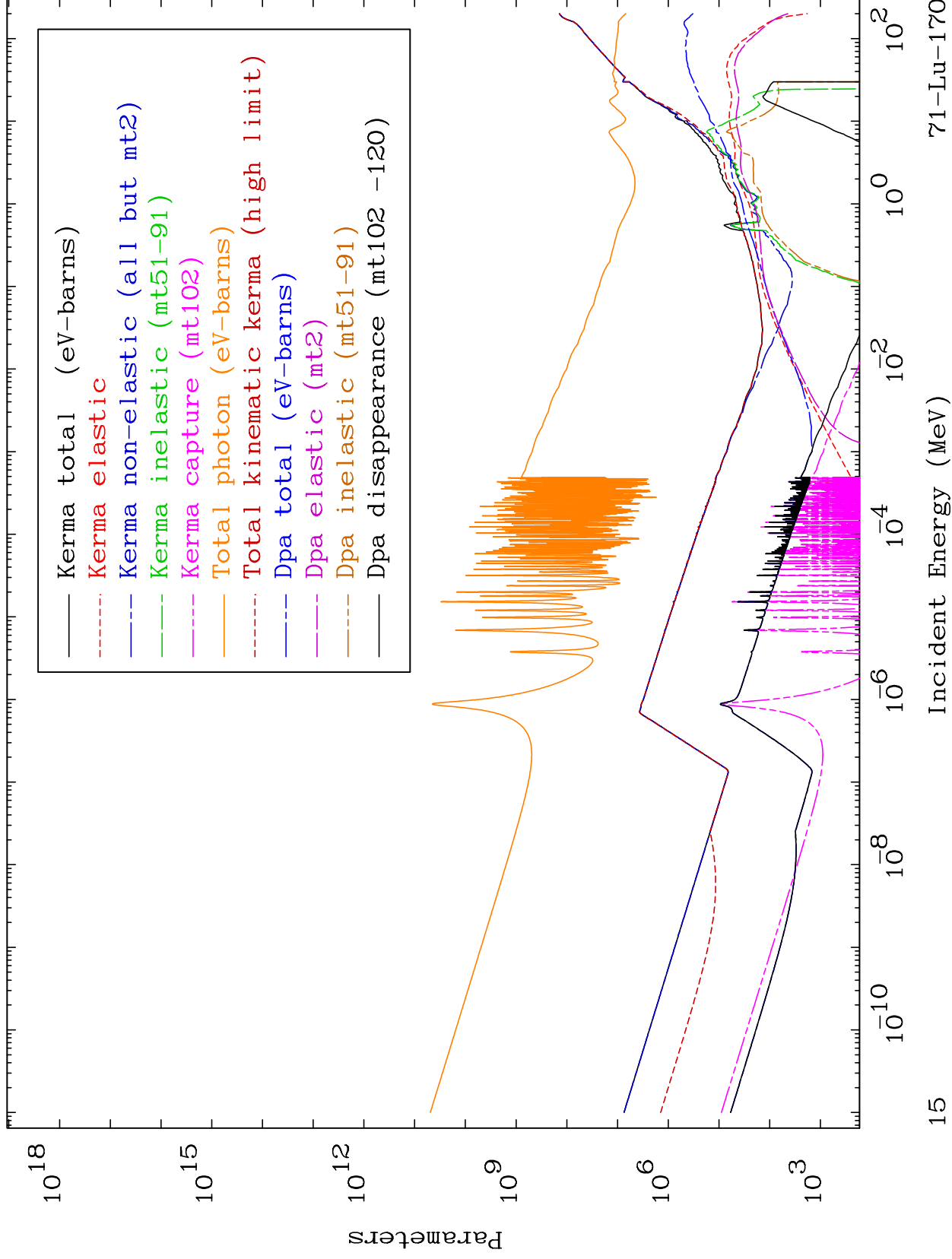
71-Lu-170

MAT 7110

(n,  $\alpha$ ) Levels  
293 Kelvin Cross Sections

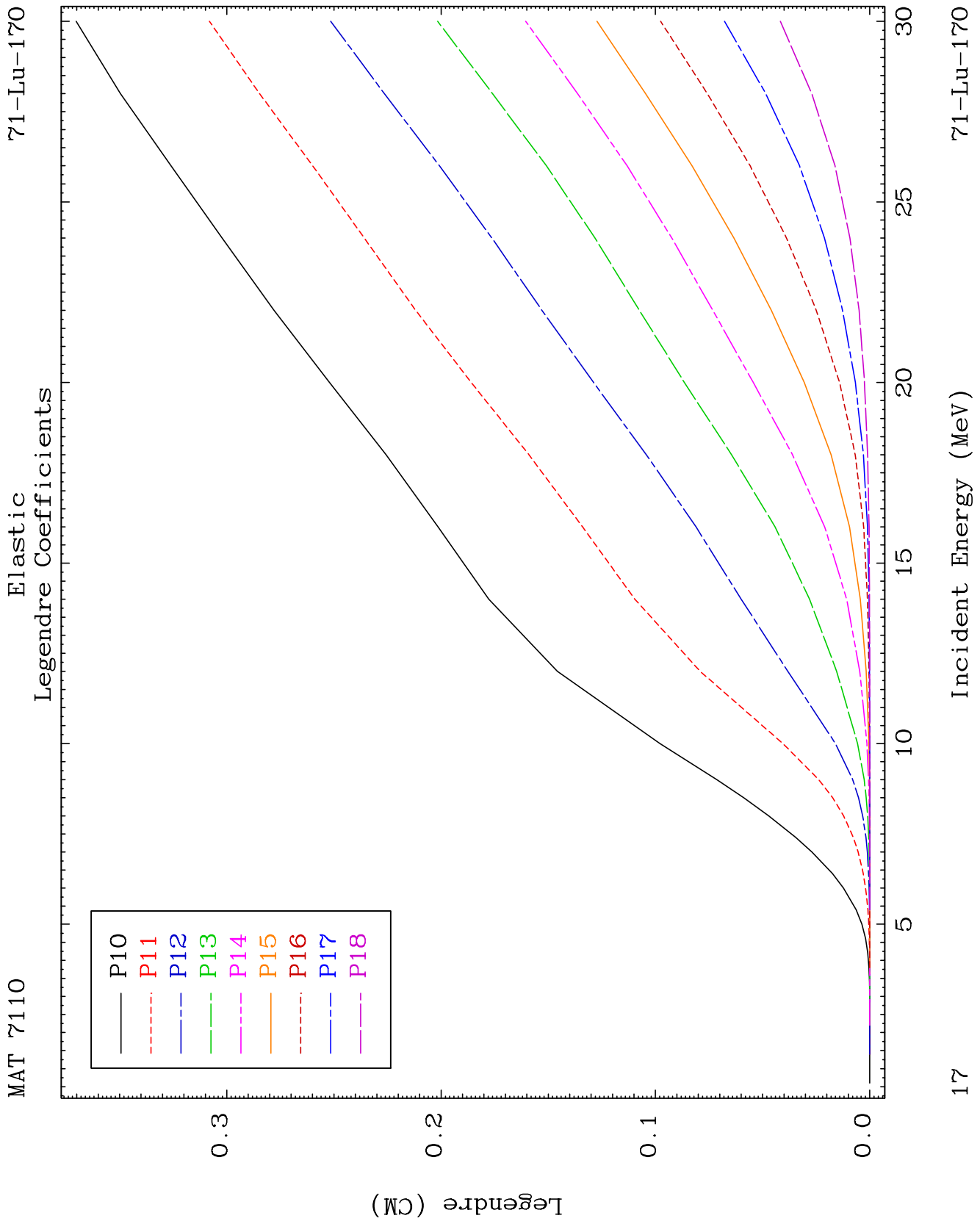
71-Lu-170







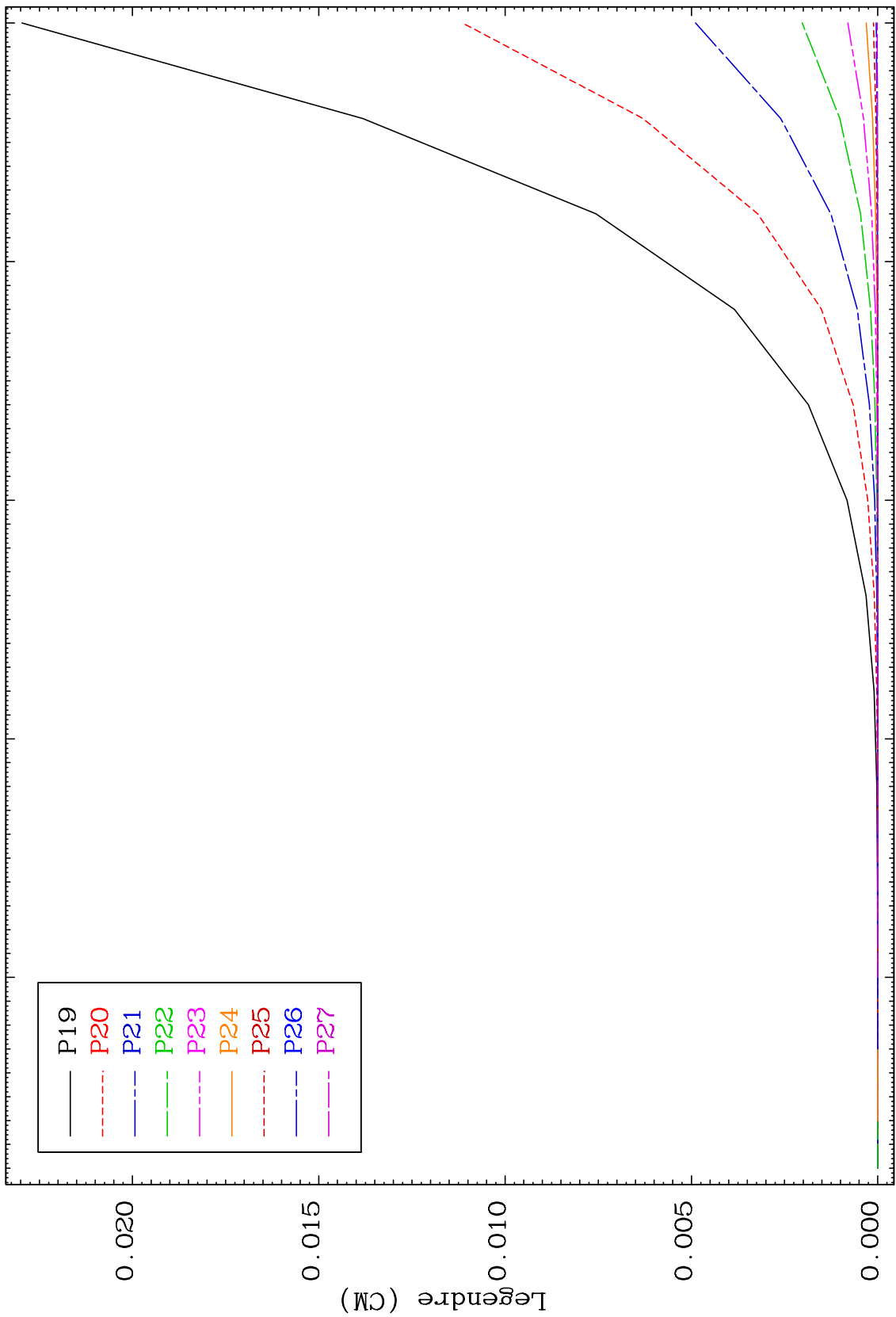




MAT 7110

Elastic  
Legendre Coefficients

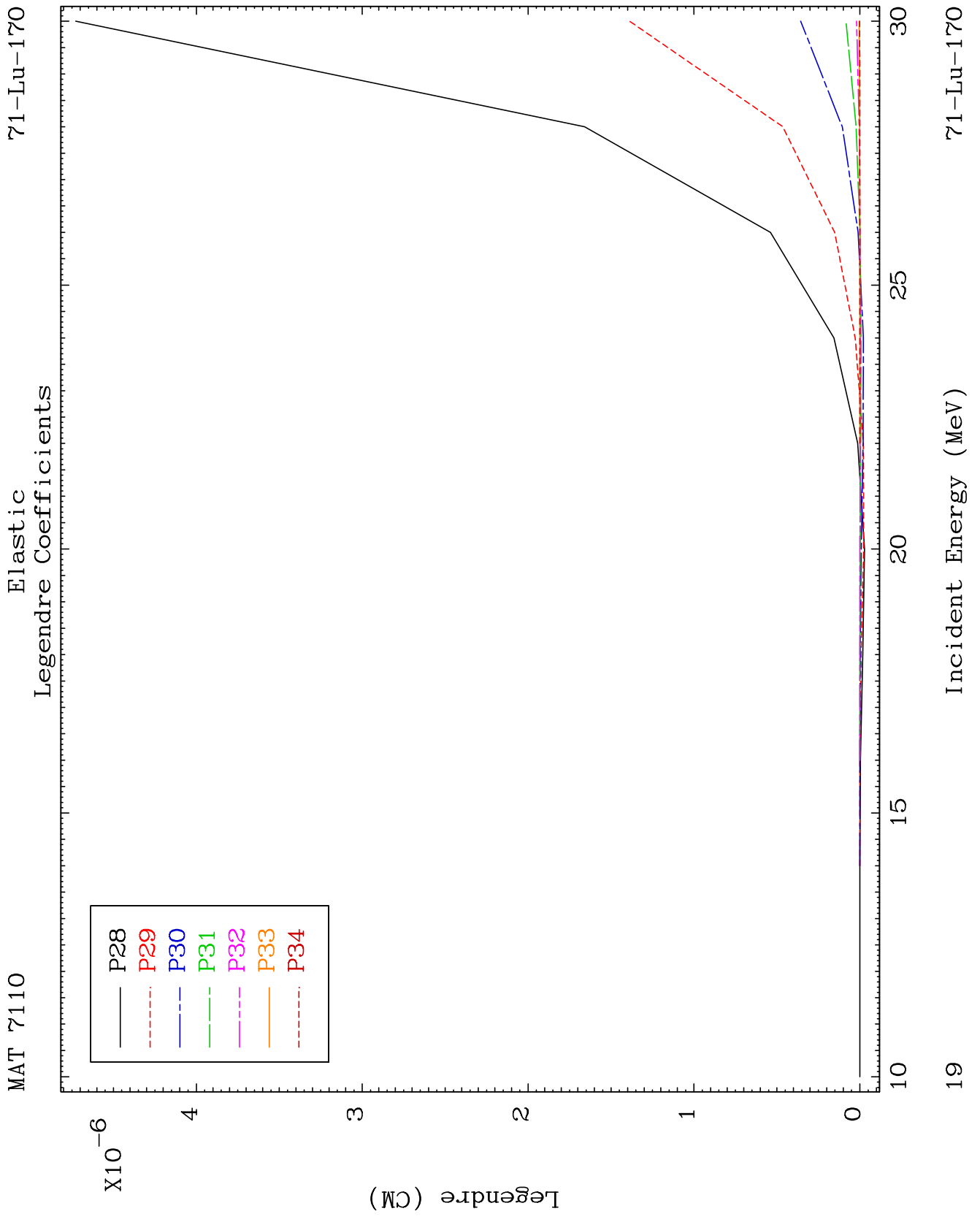
71-Lu-170



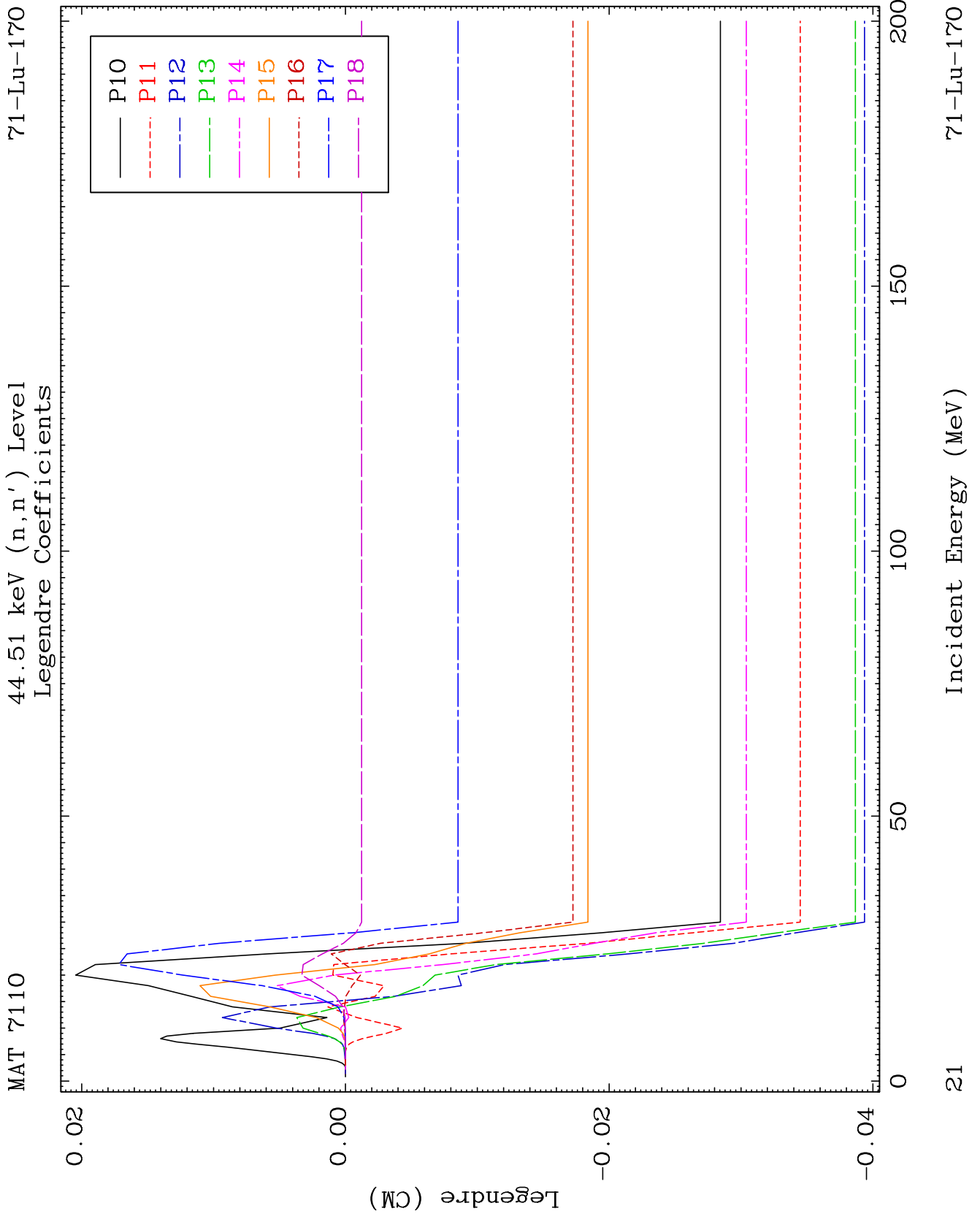
18

Incident Energy (MeV)

71-Lu-170



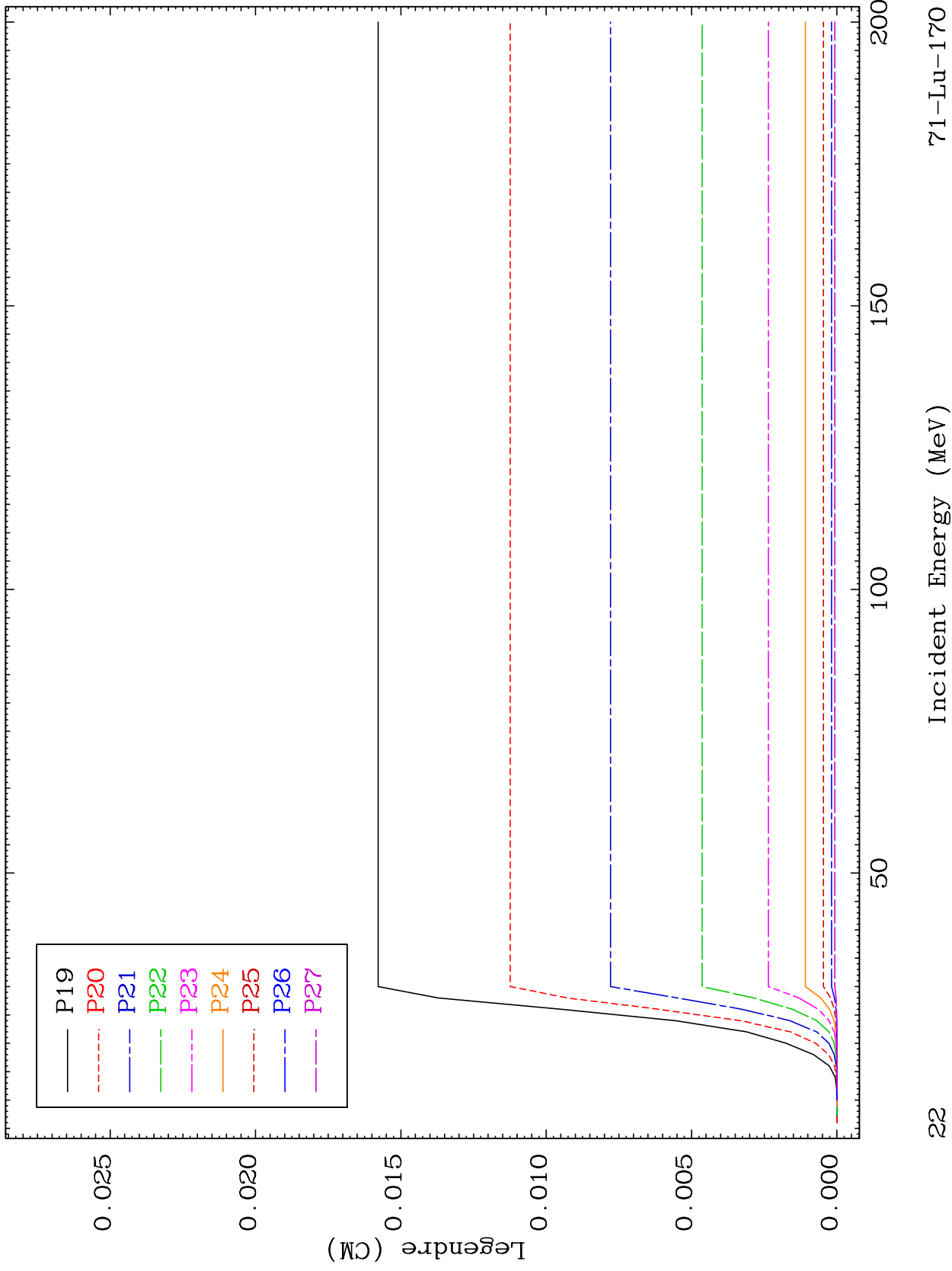




MAT 7110

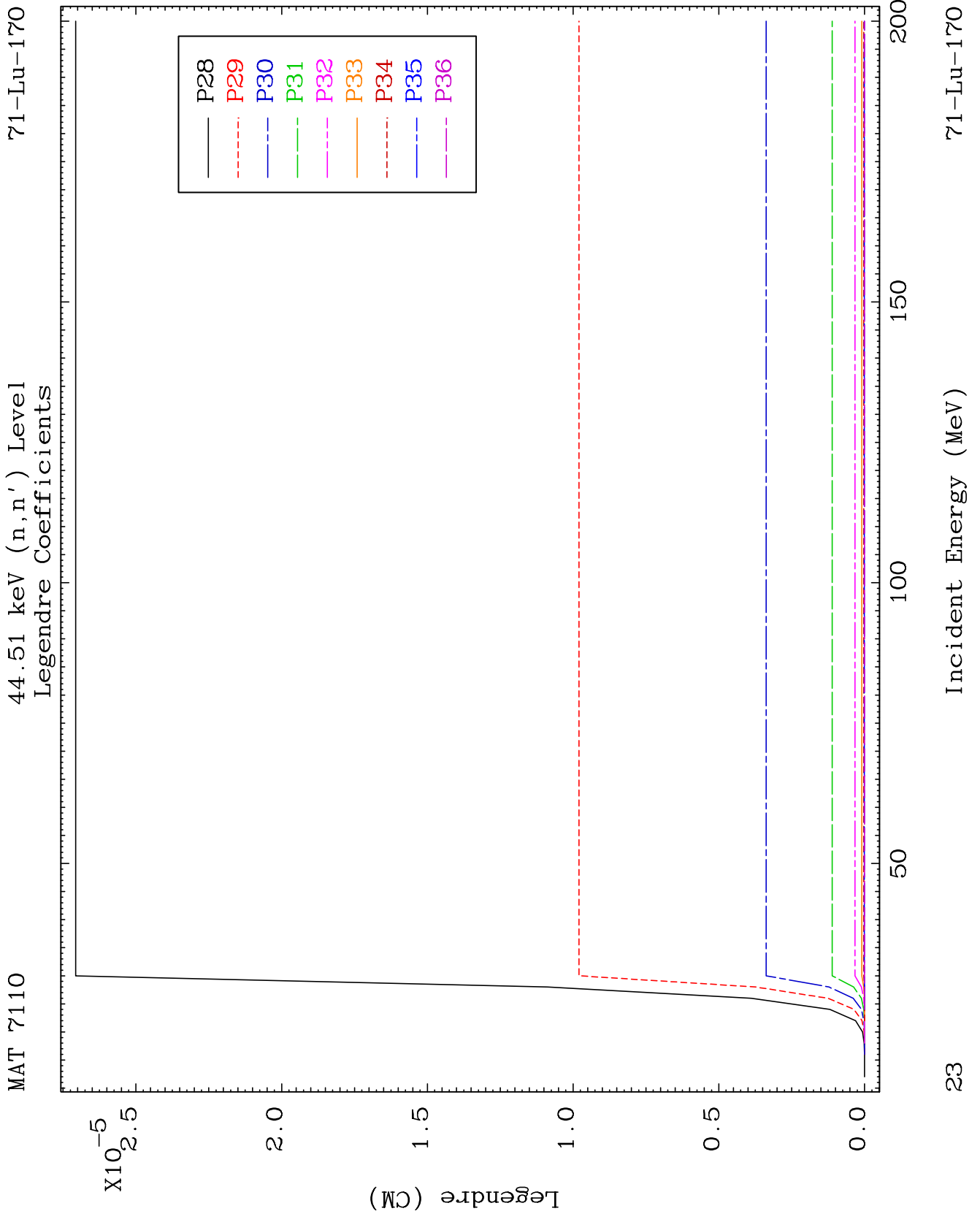
44.51 keV (n,n') Level  
Legendre Coefficients

71-Lu-170



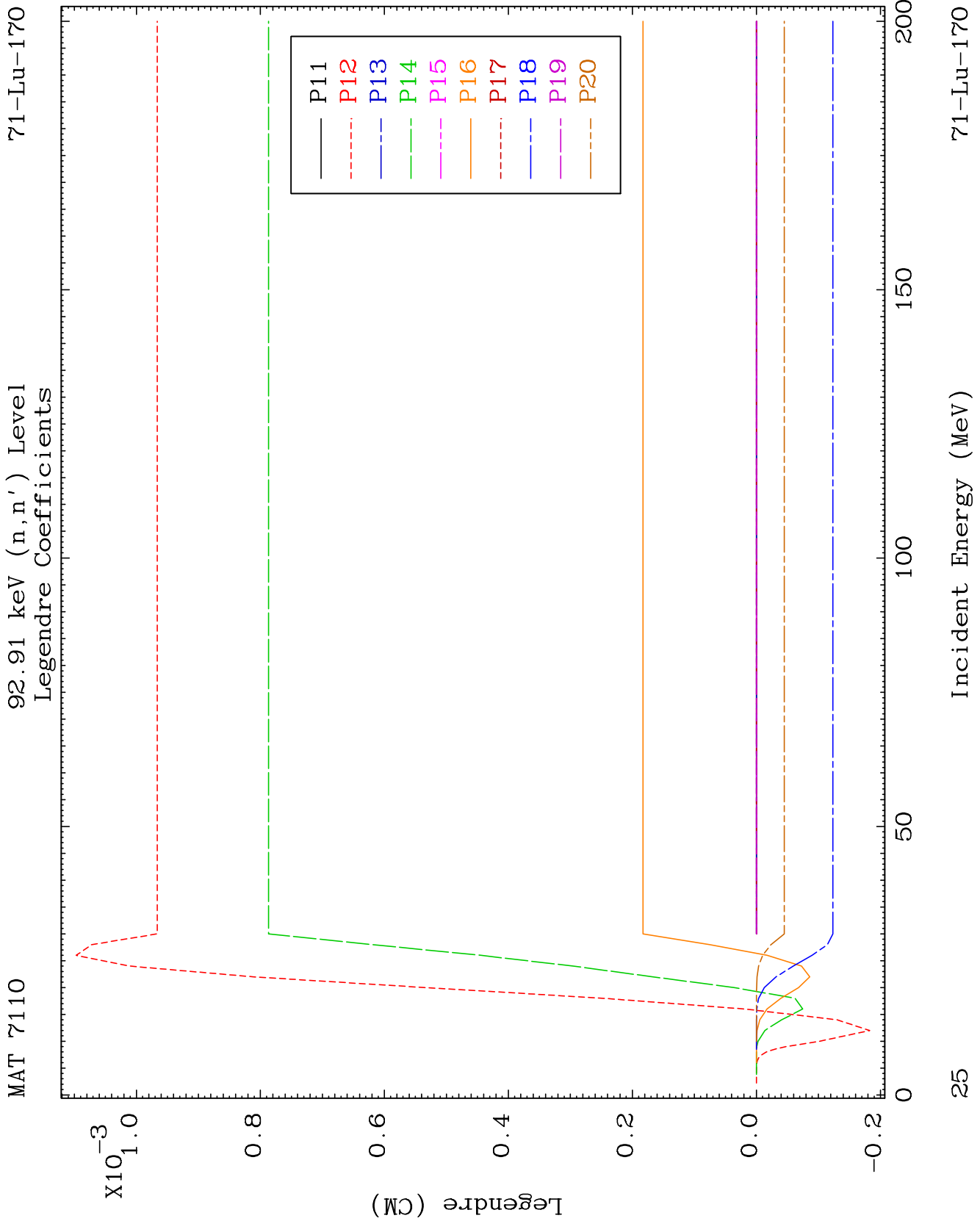
22

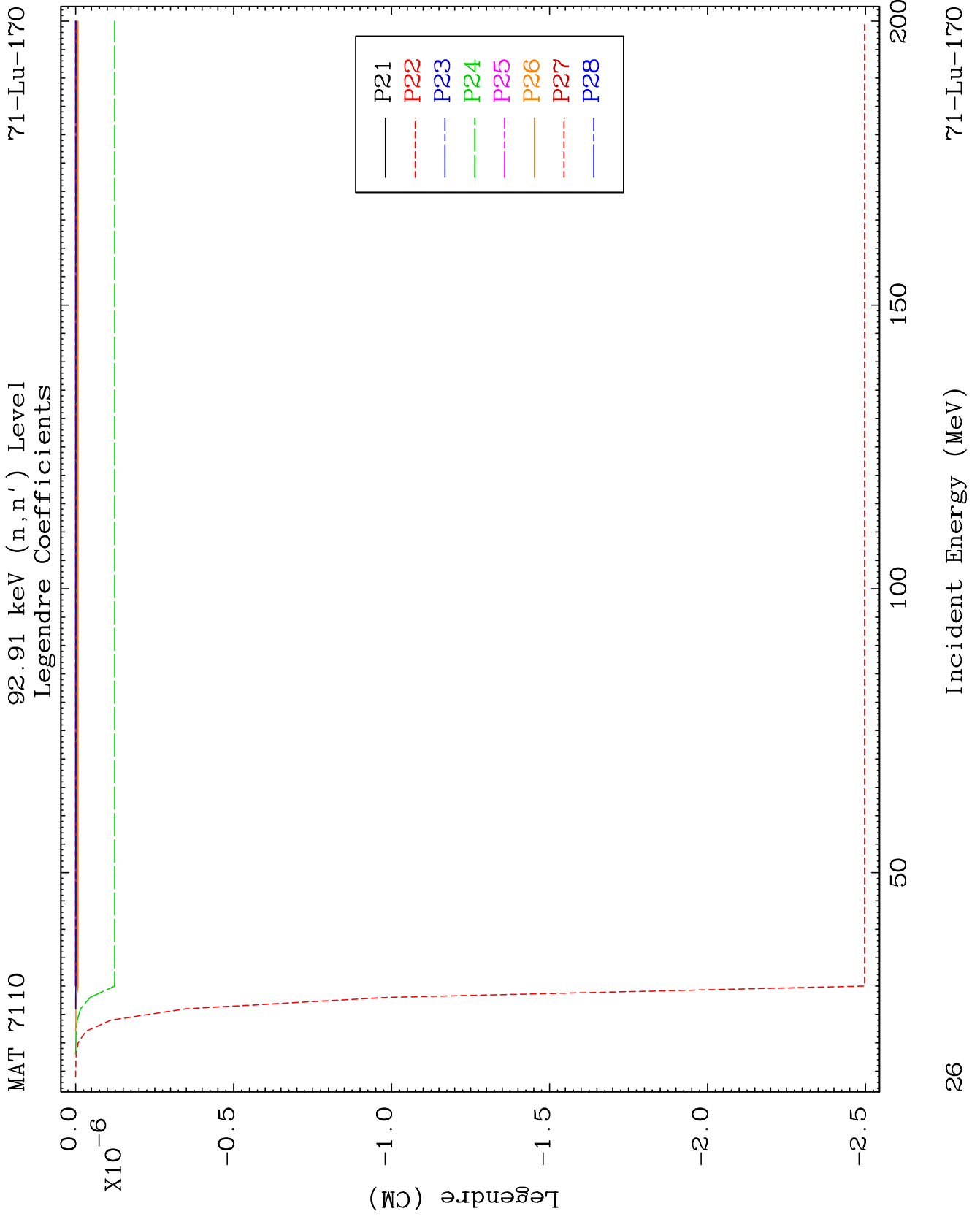
71-Lu-170

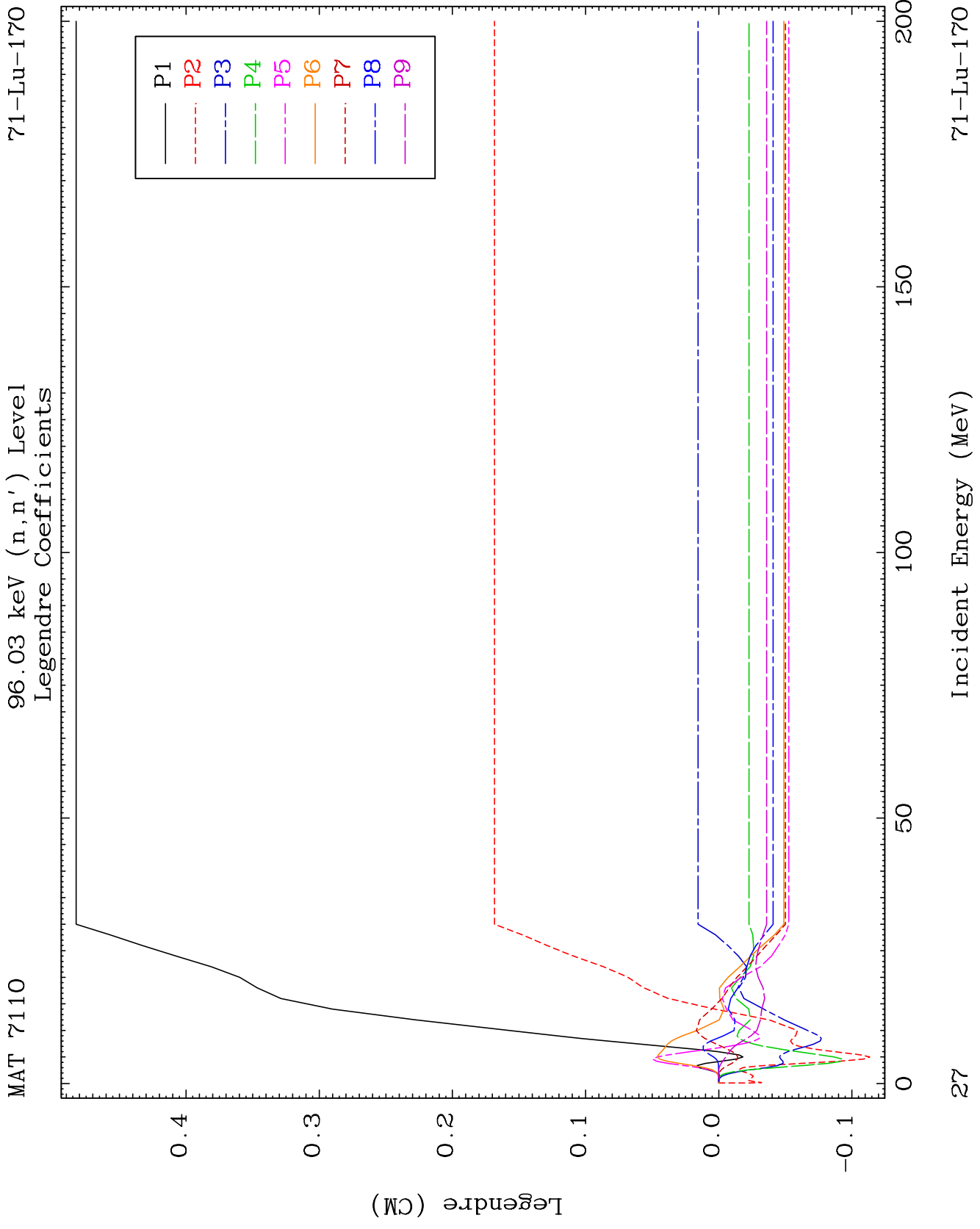


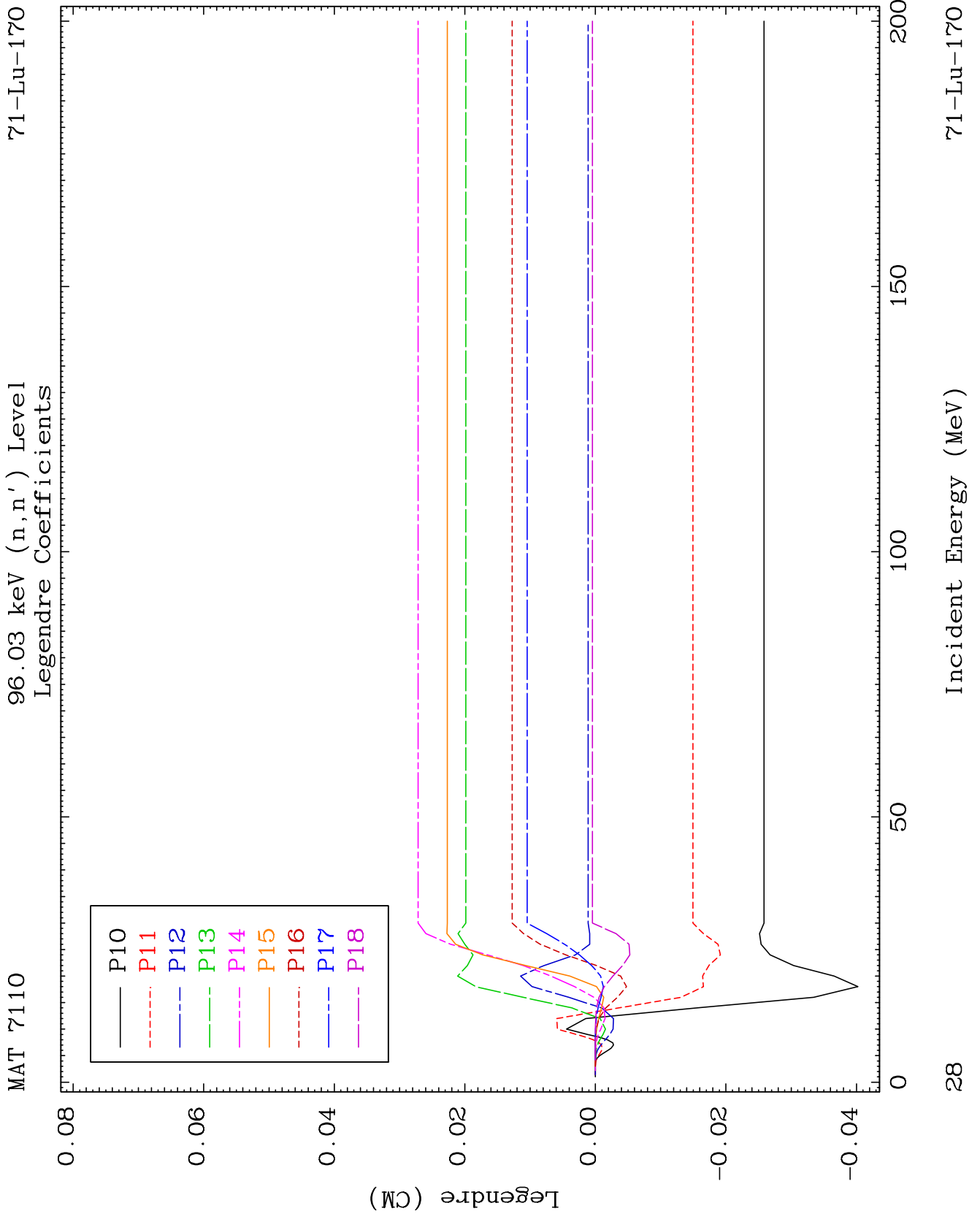








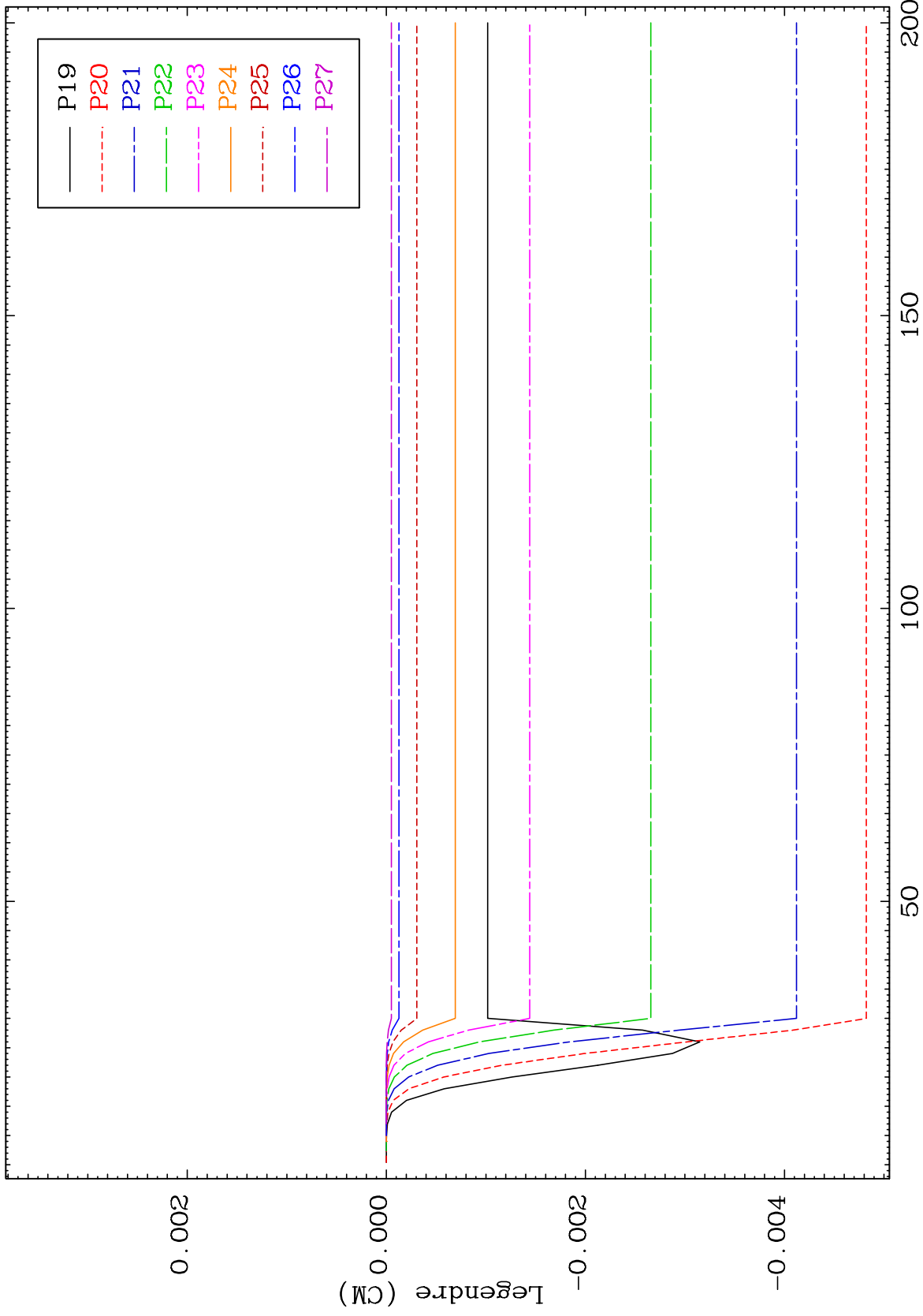




MAT 7110

96.03 keV (n,n') Level  
Legendre Coefficients

71-Lu-170

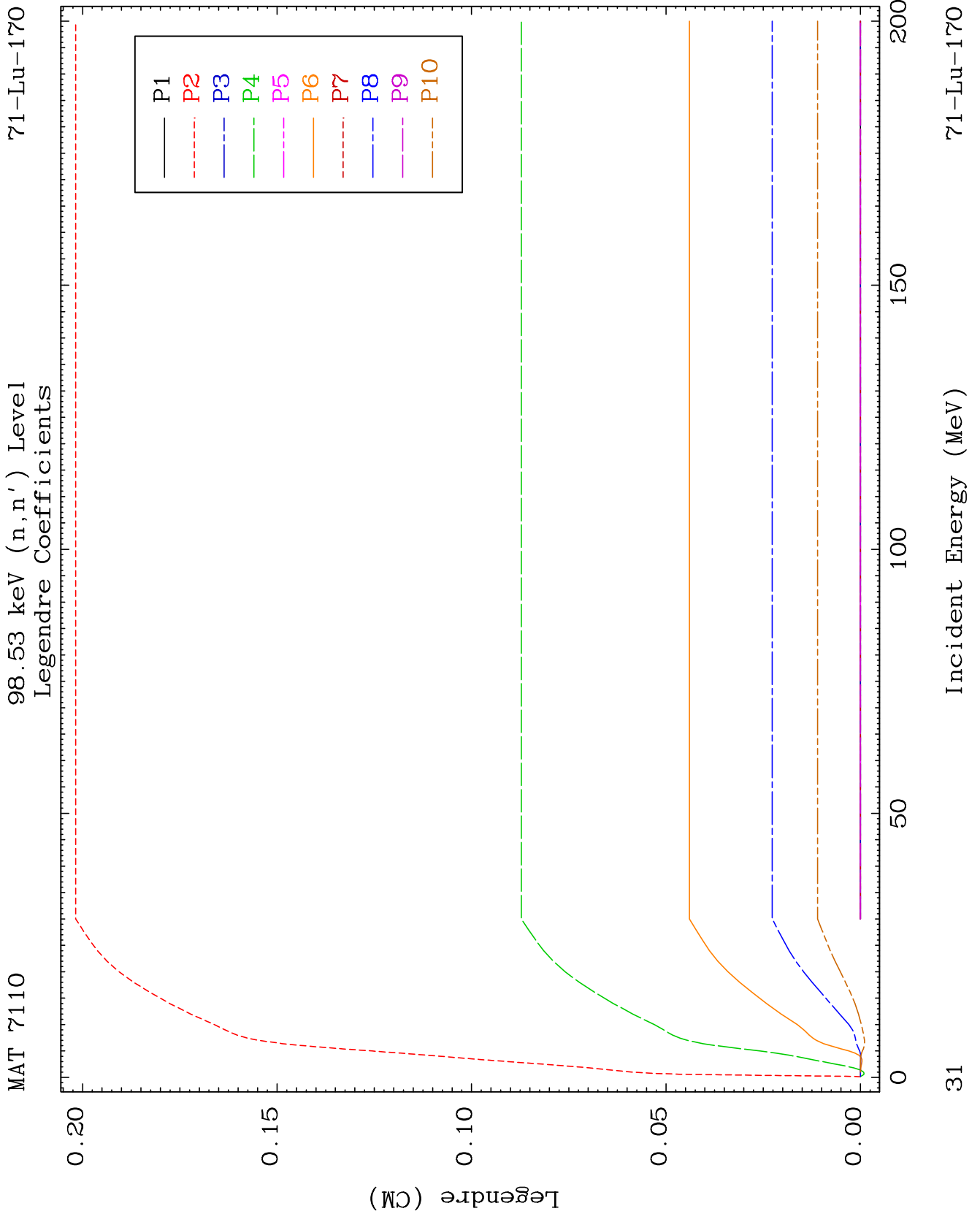


29

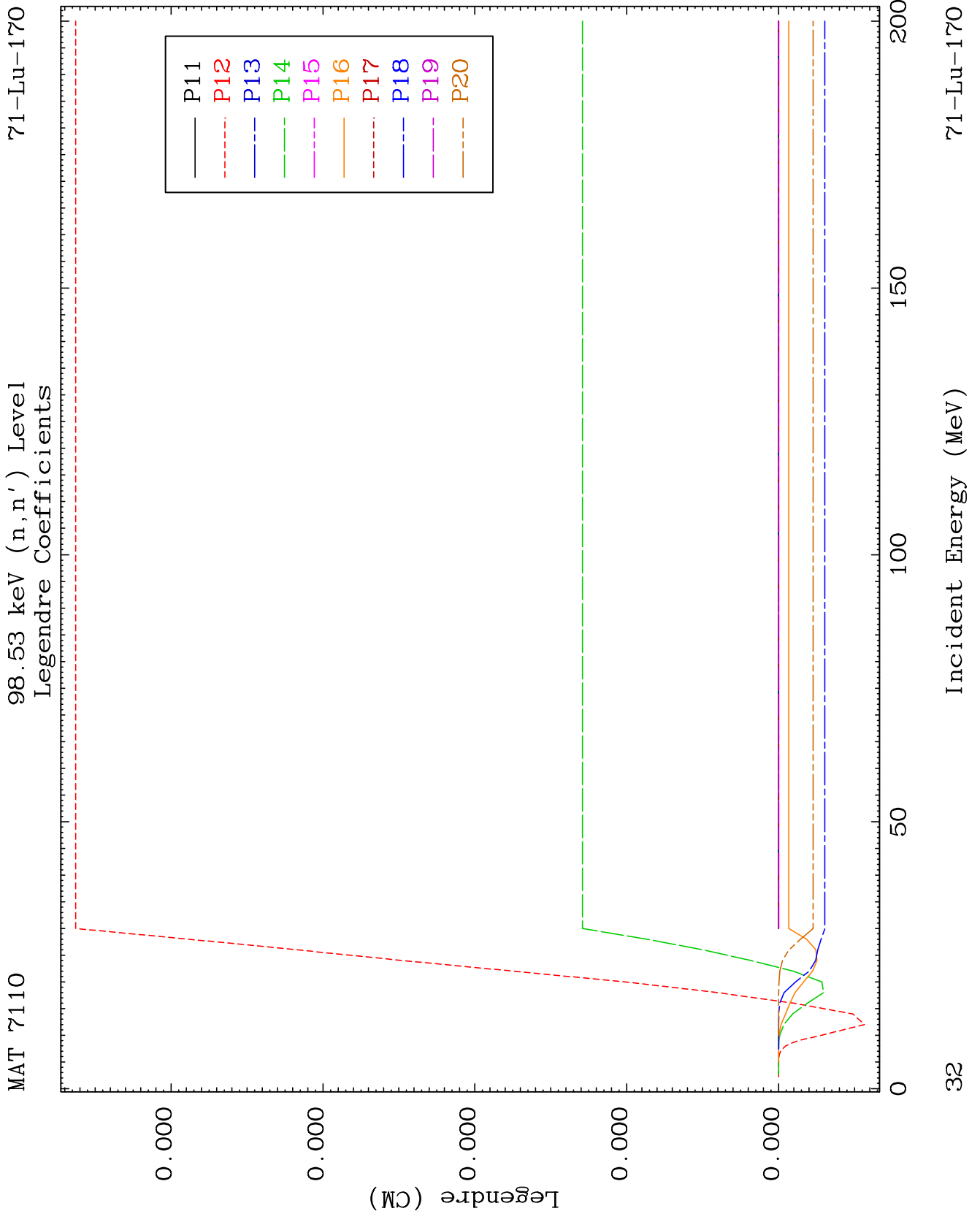
Incident Energy (MeV)

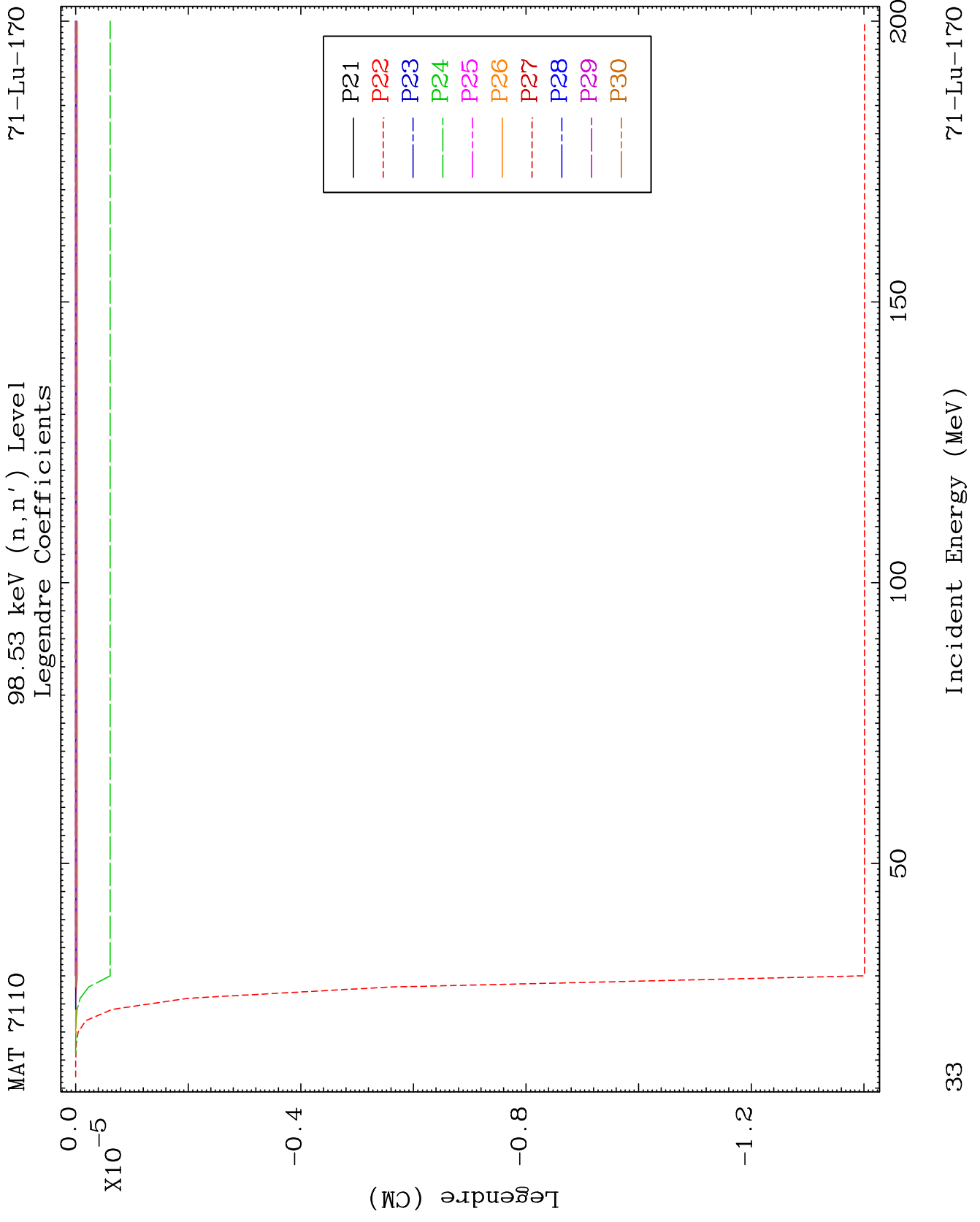
71-Lu-170







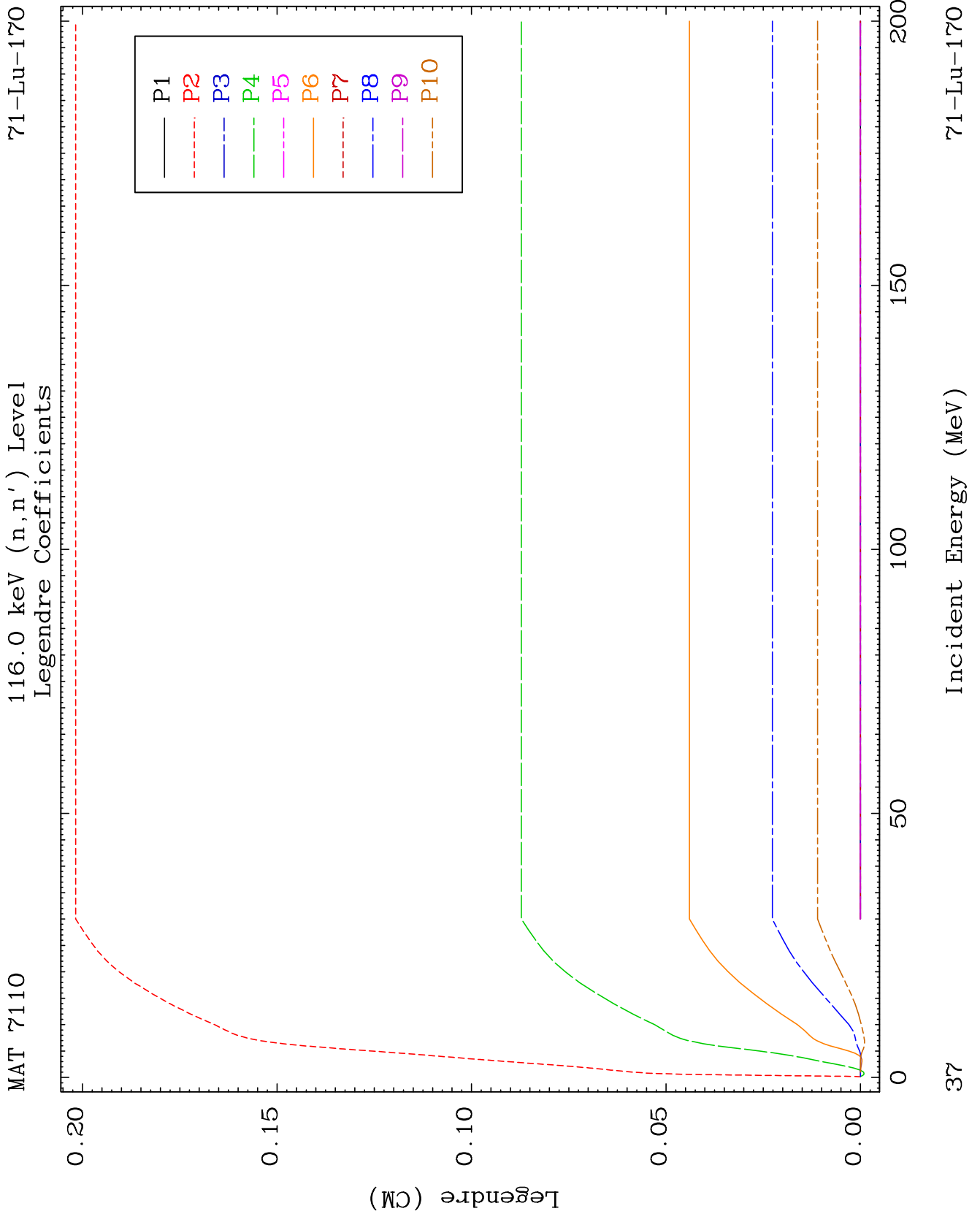




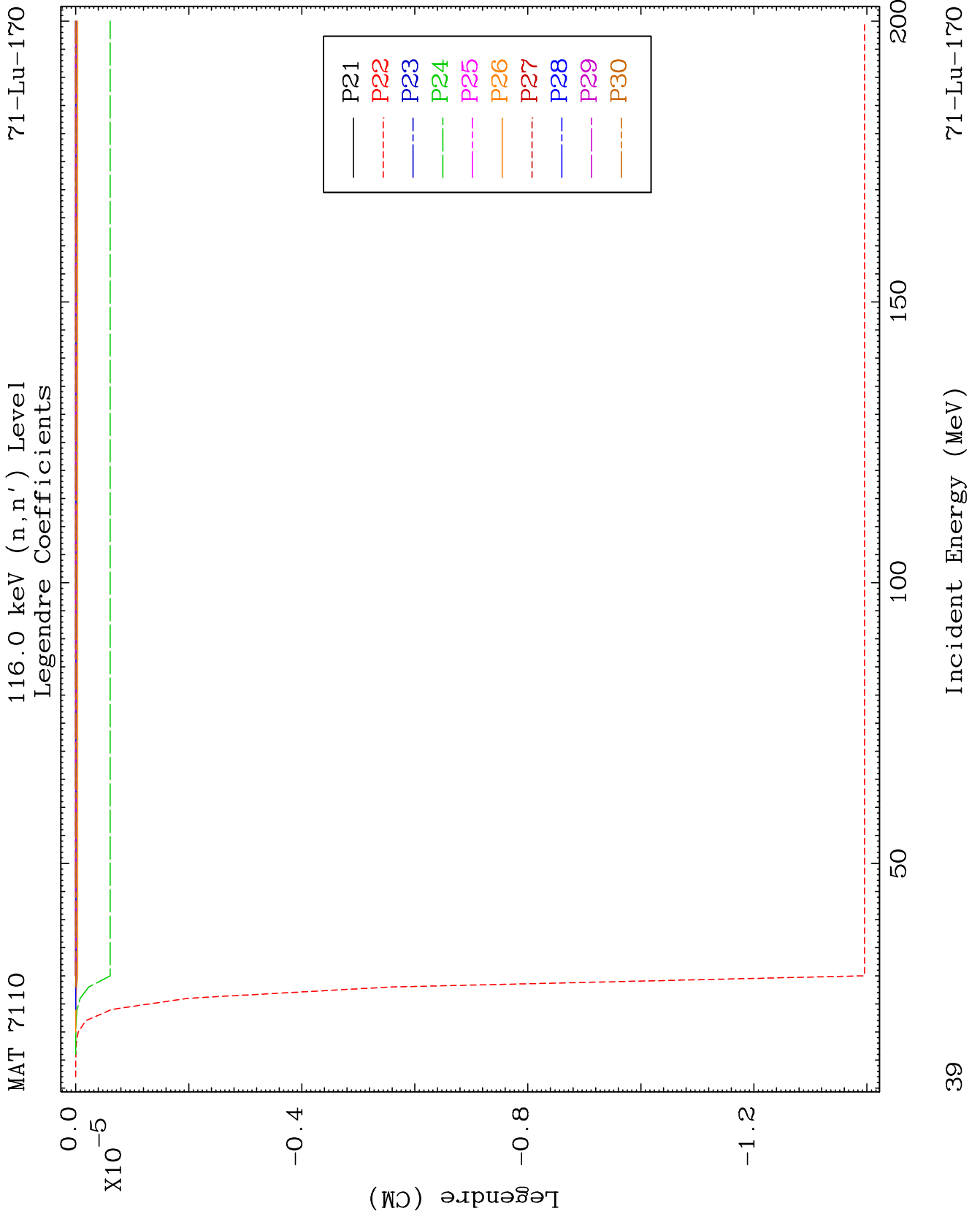




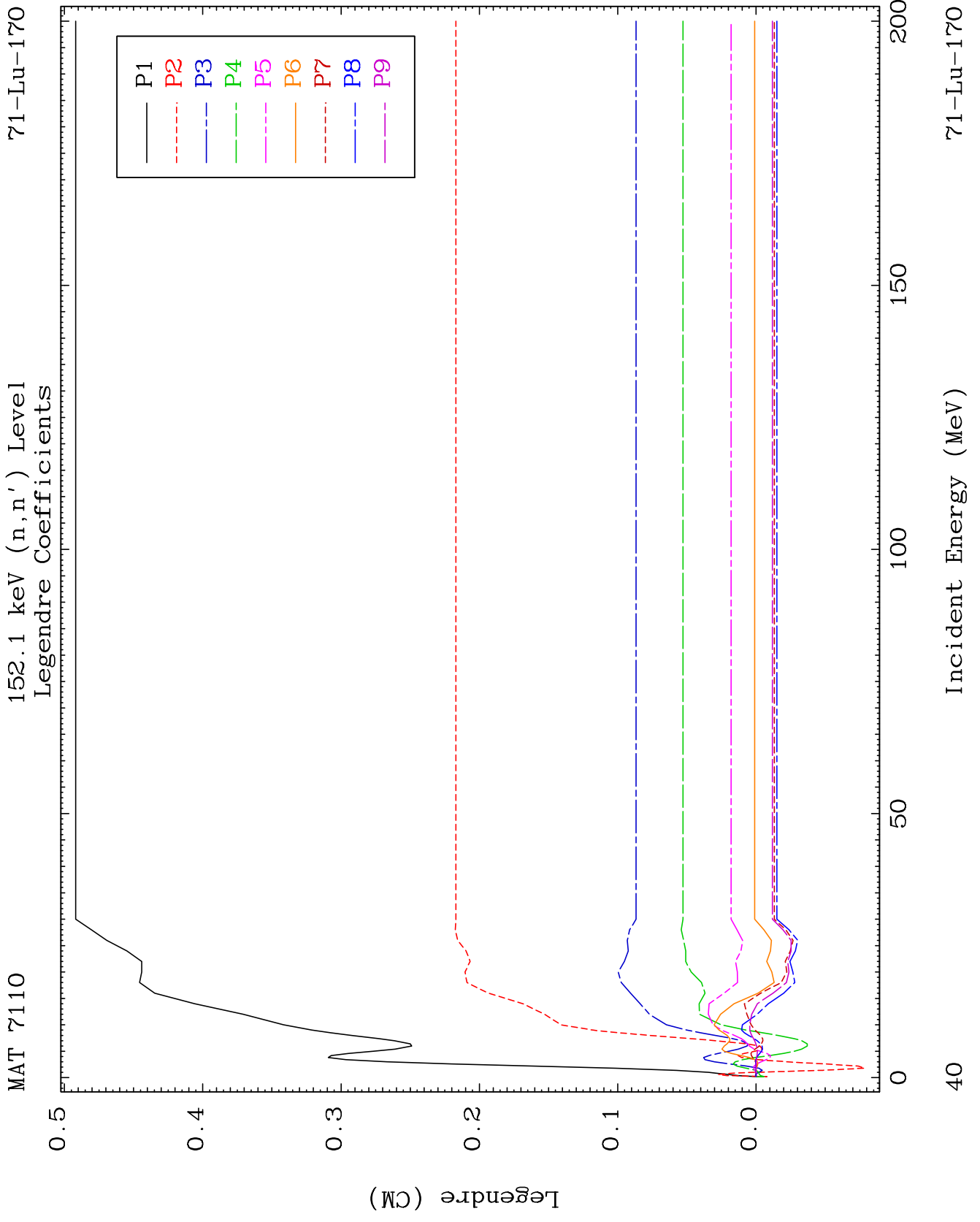








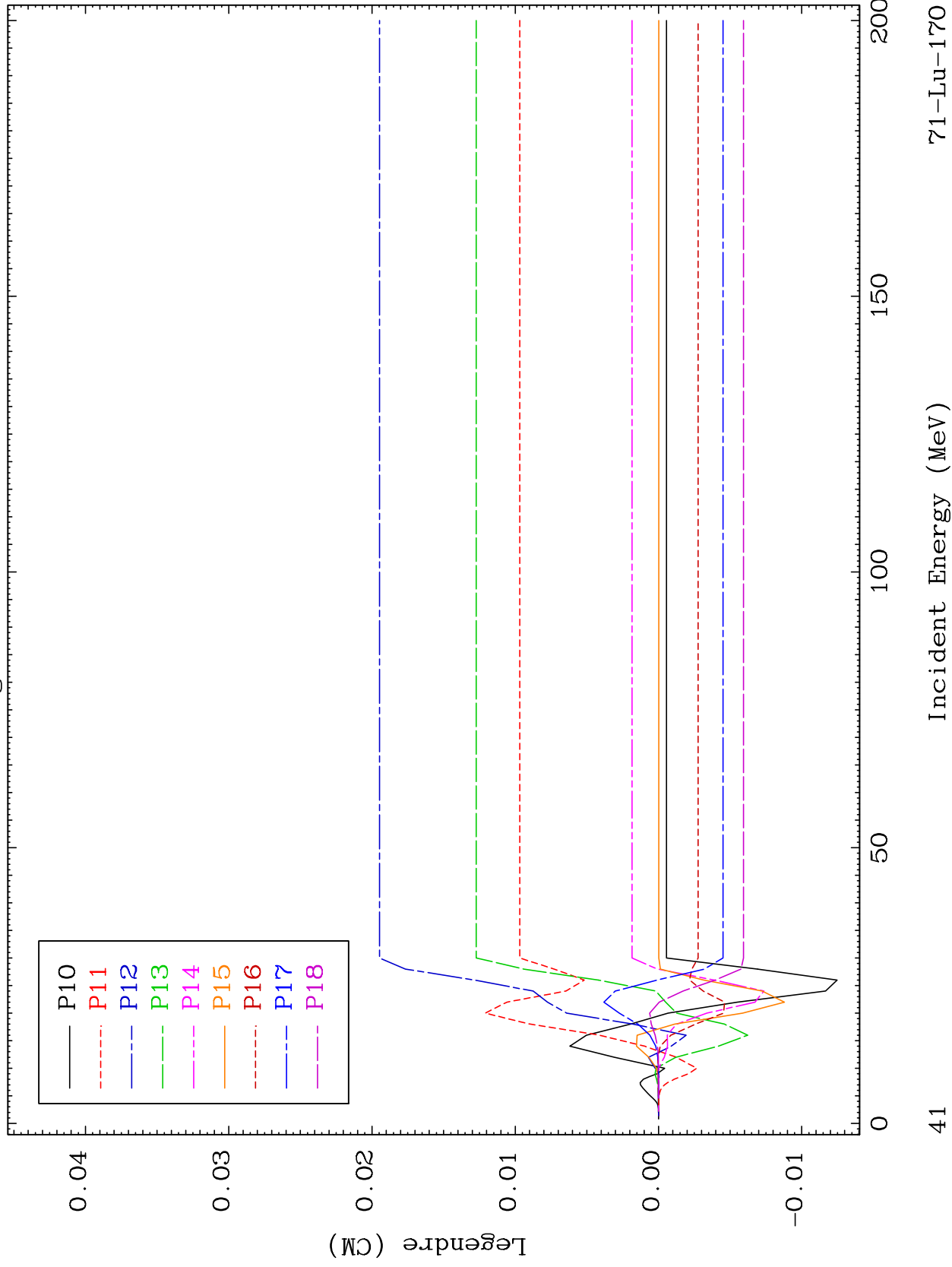




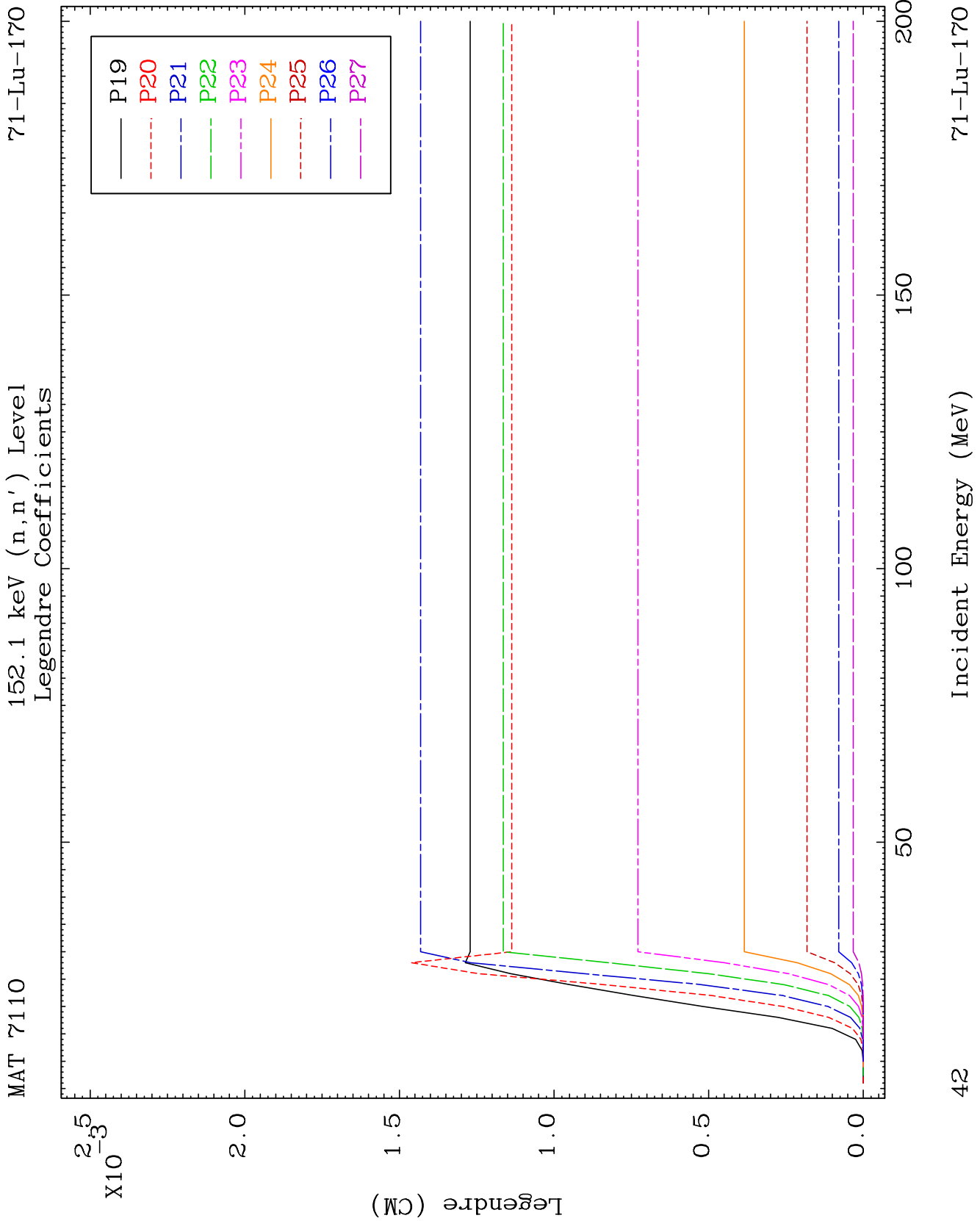
MAT 7110

152.1 keV (n,n') Level  
Legendre Coefficients

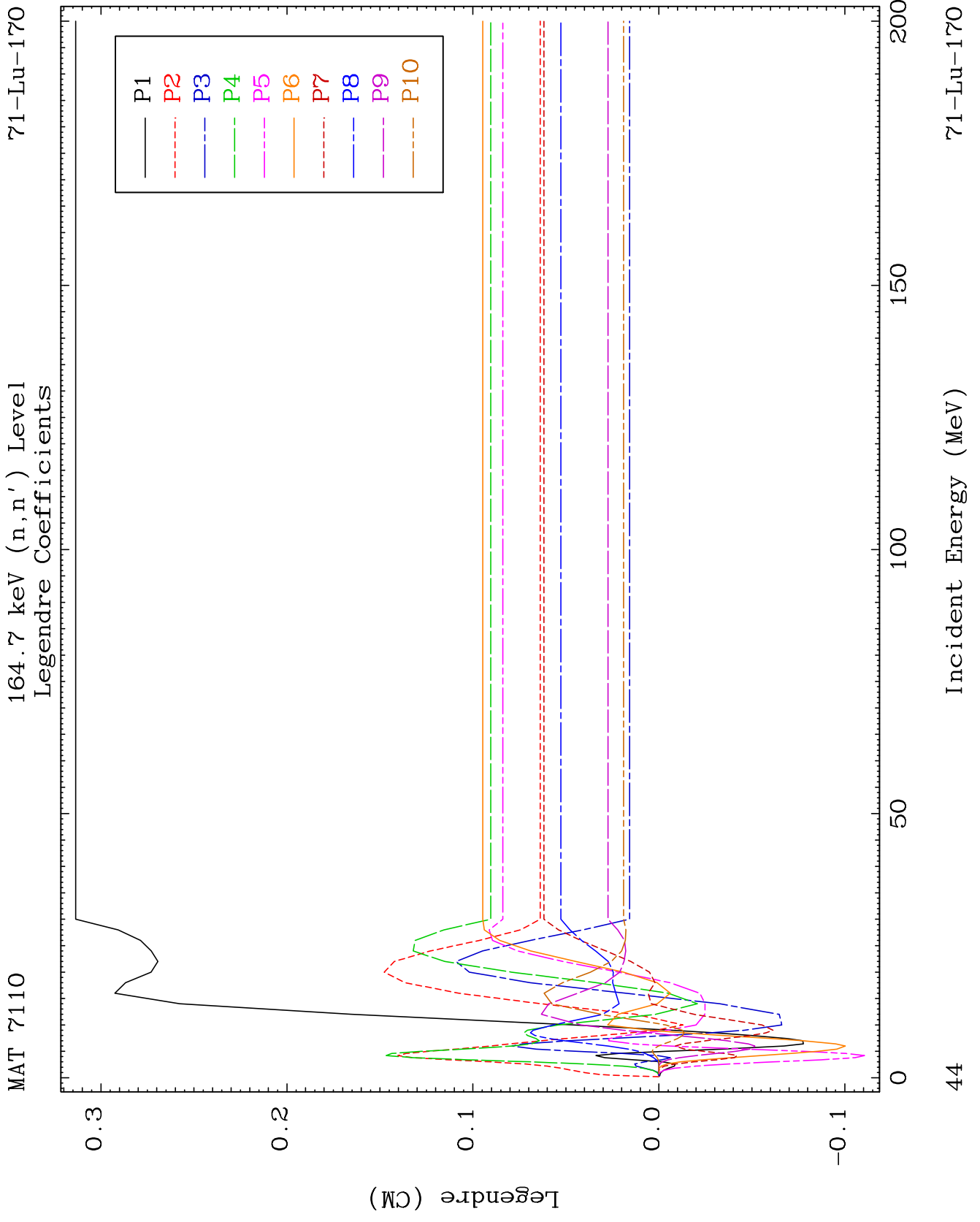
71-Lu-170

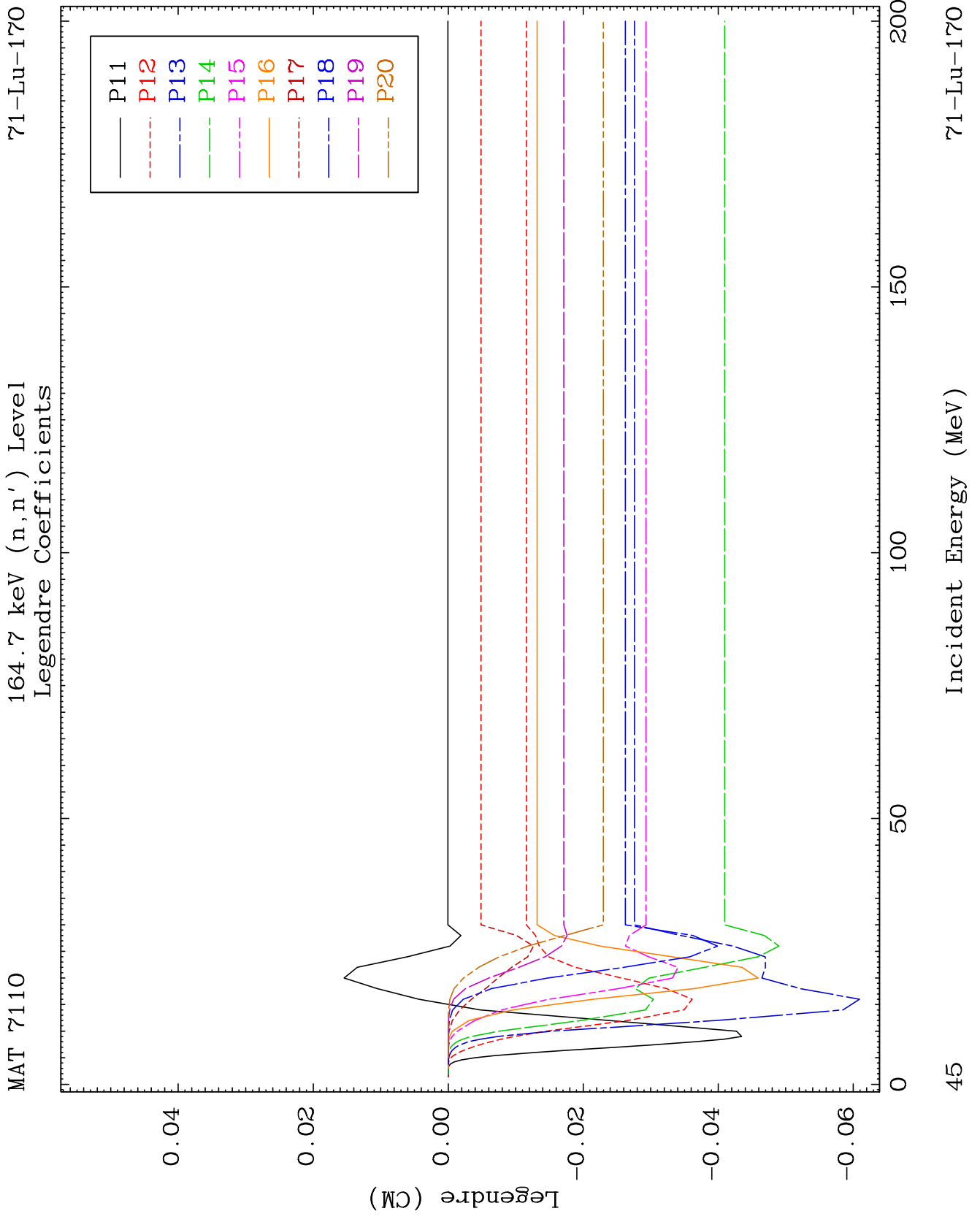


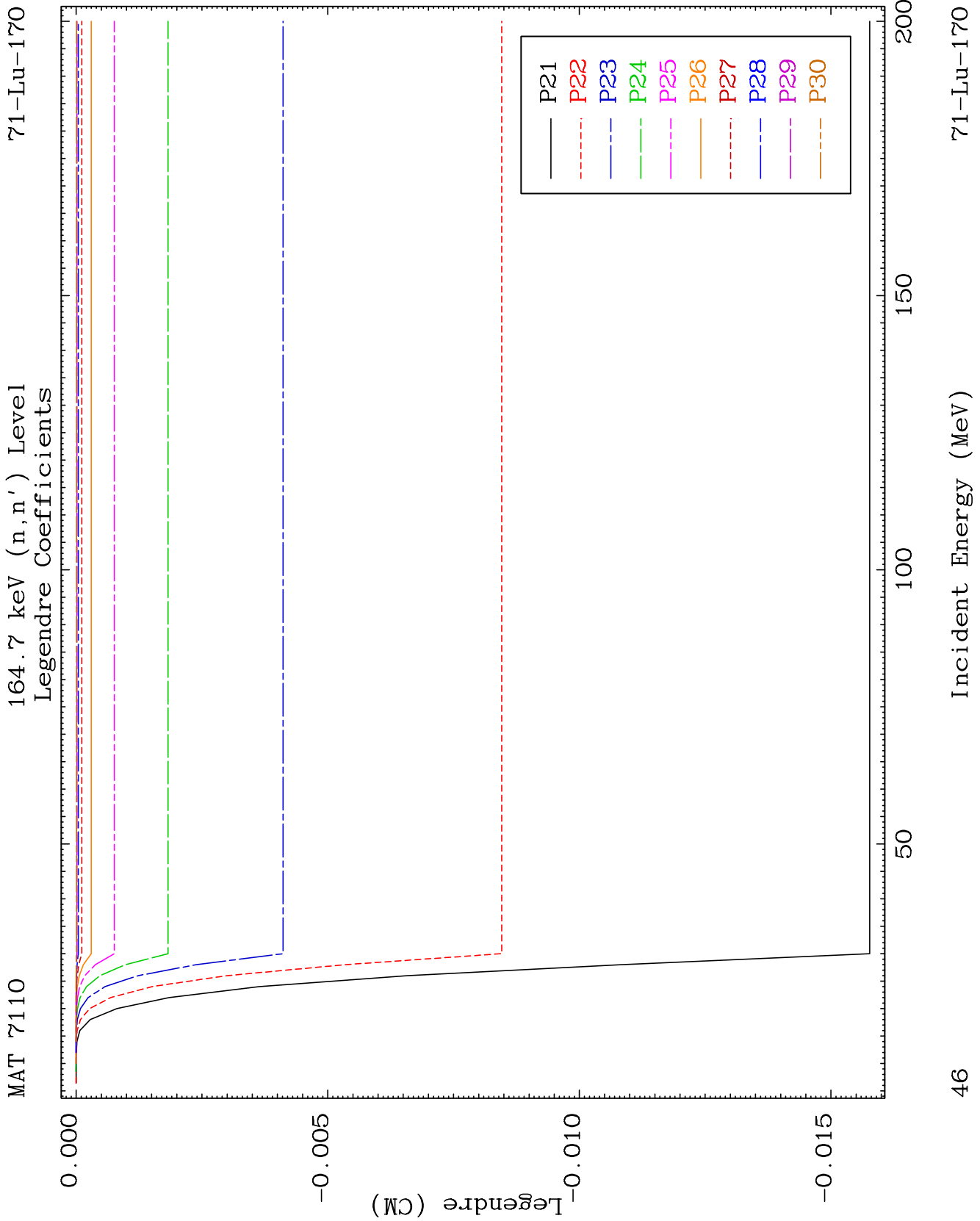
41

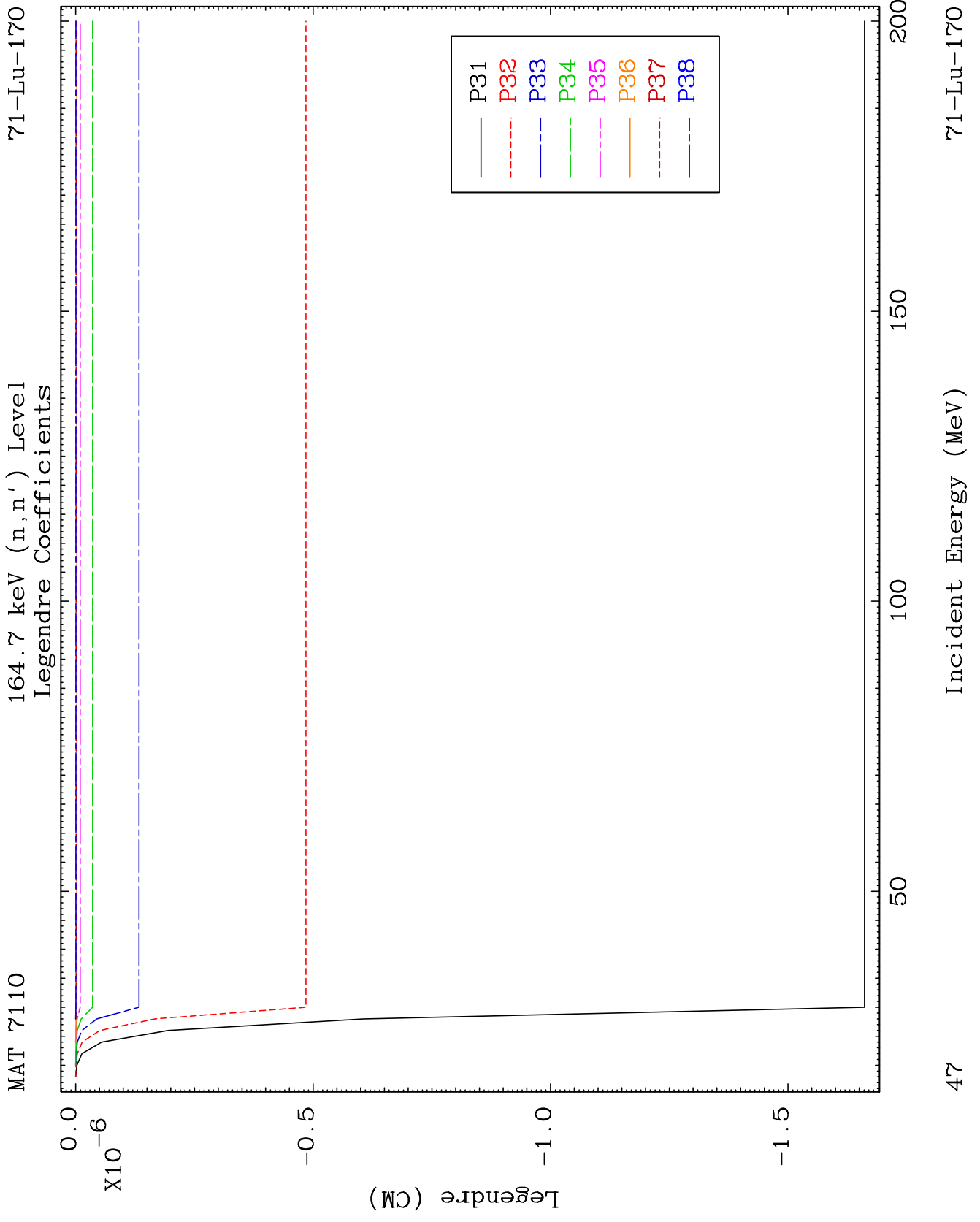




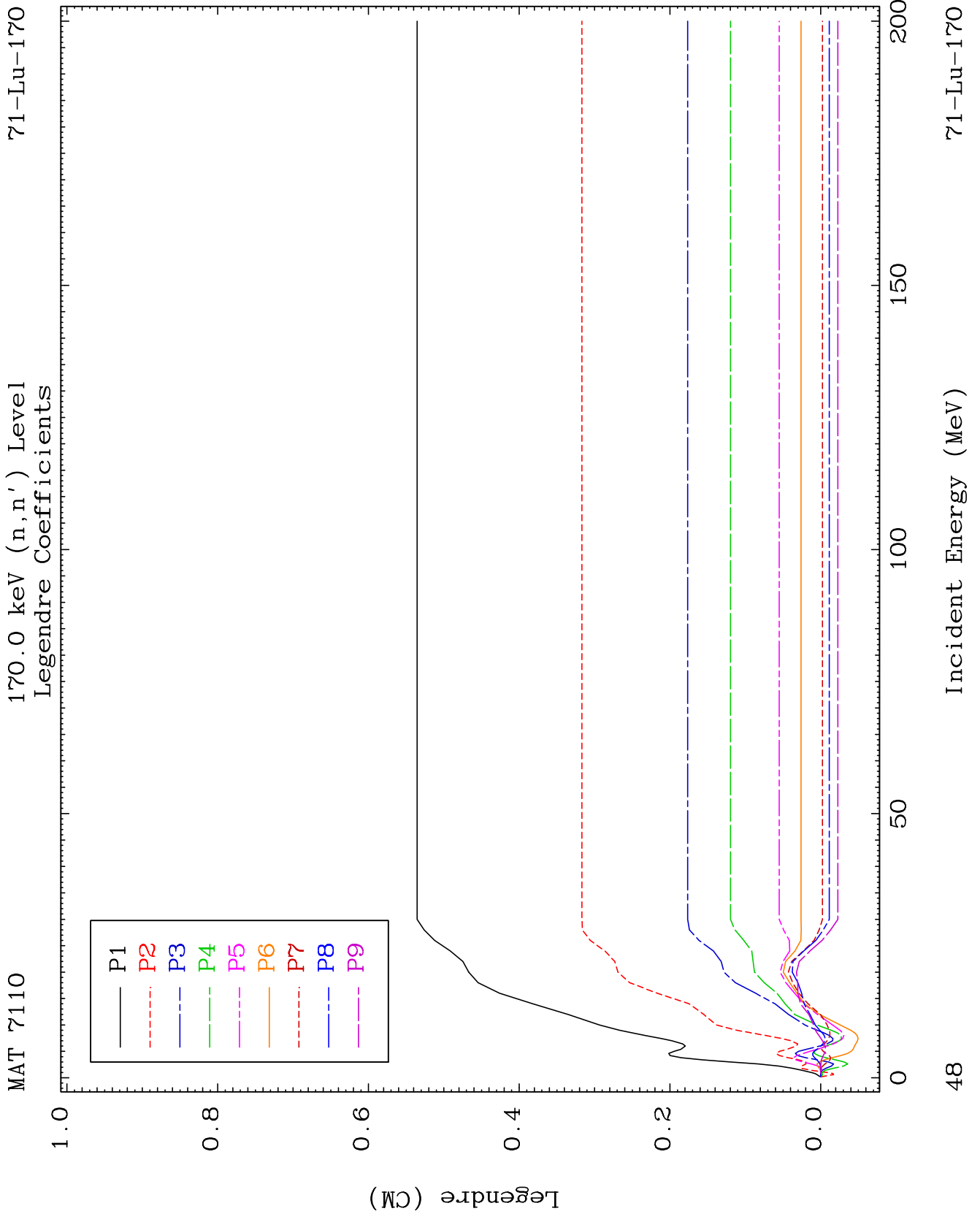


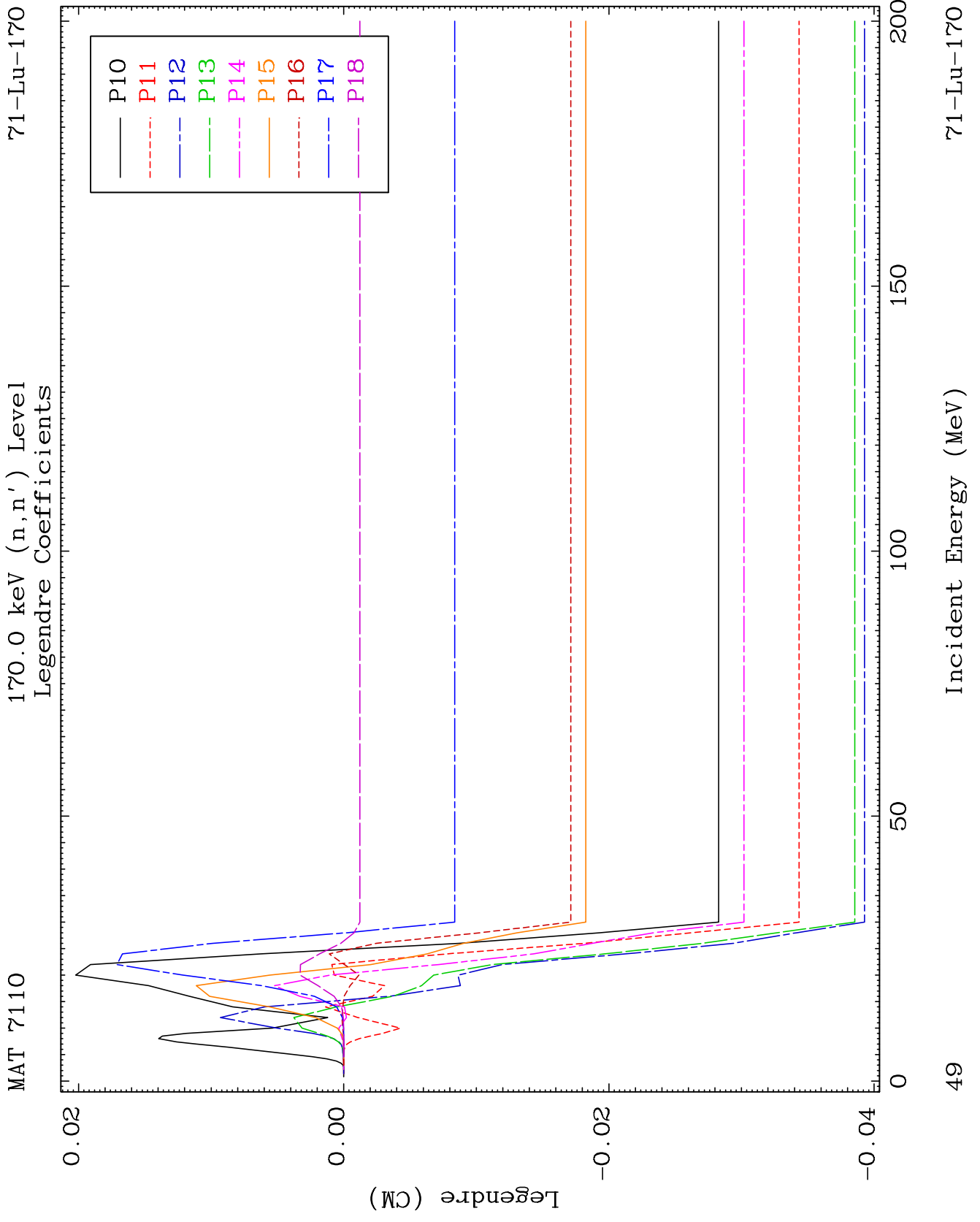


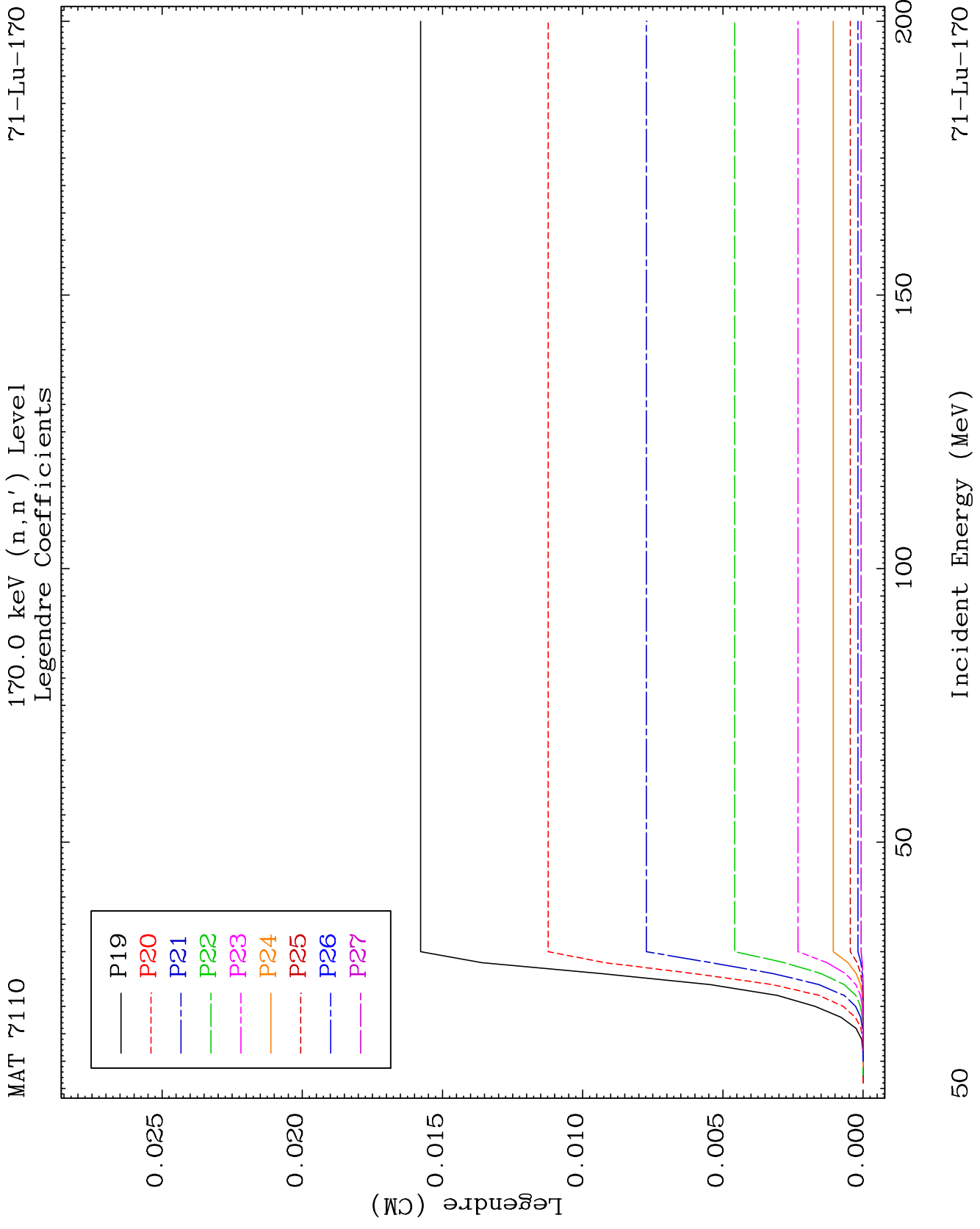






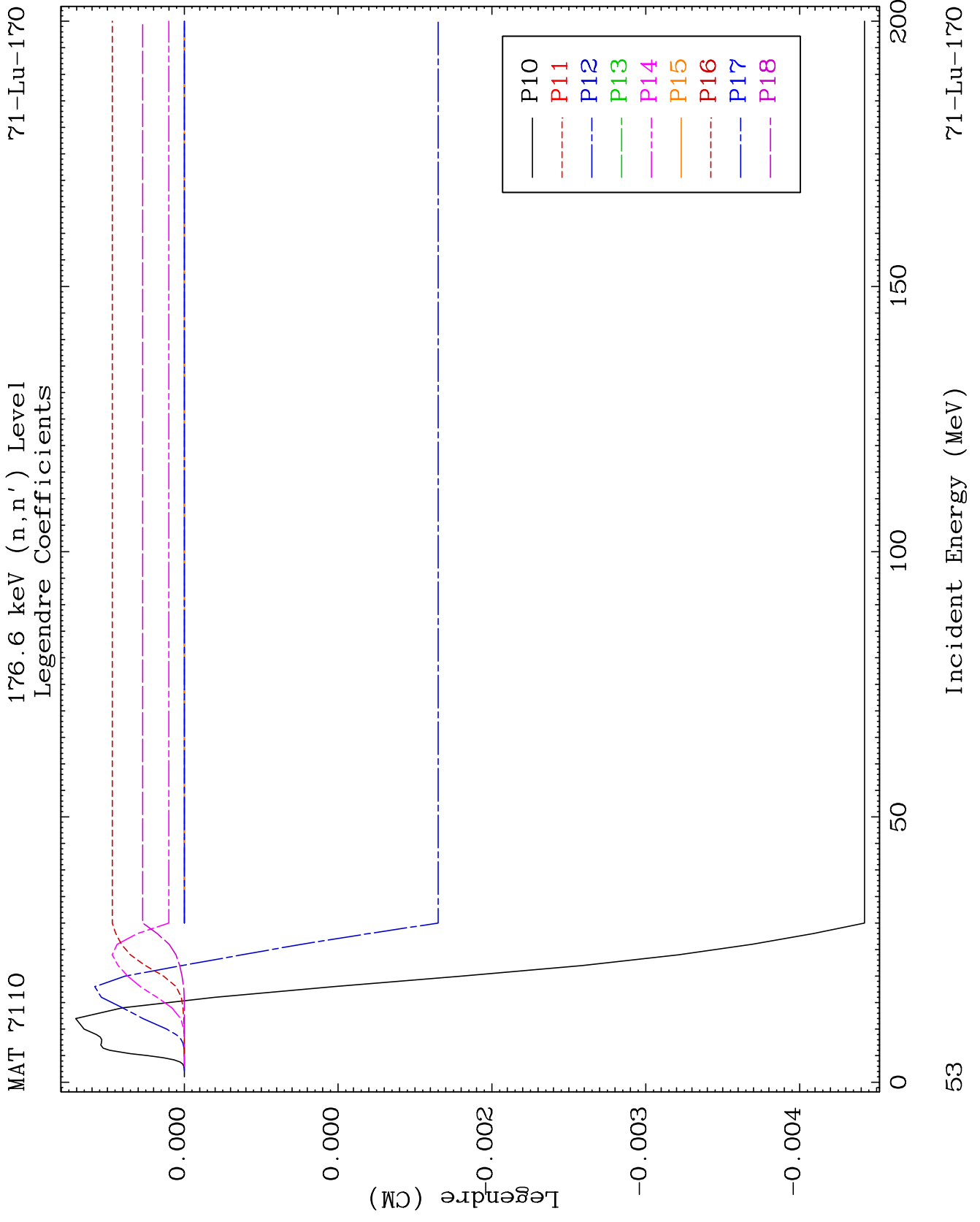




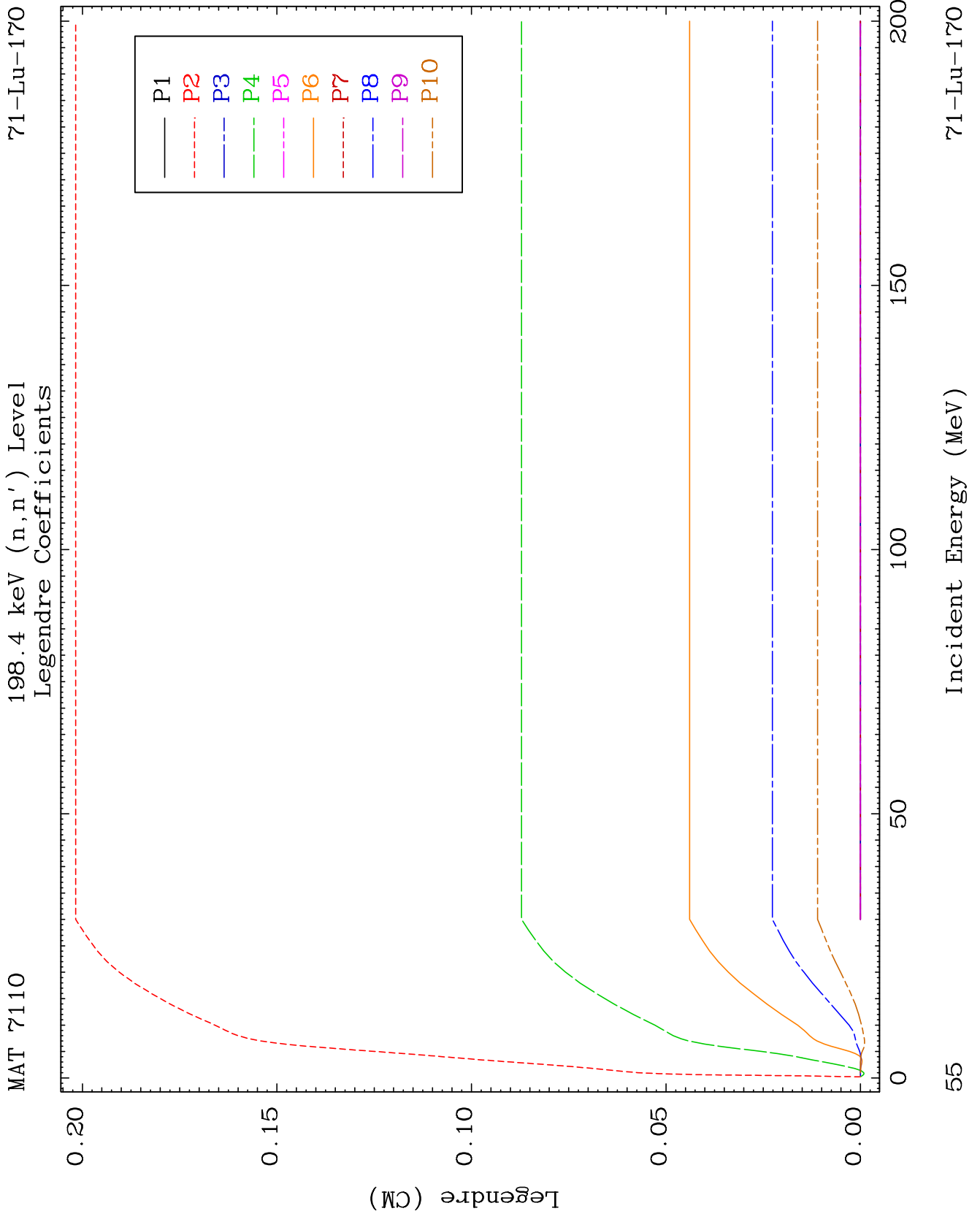




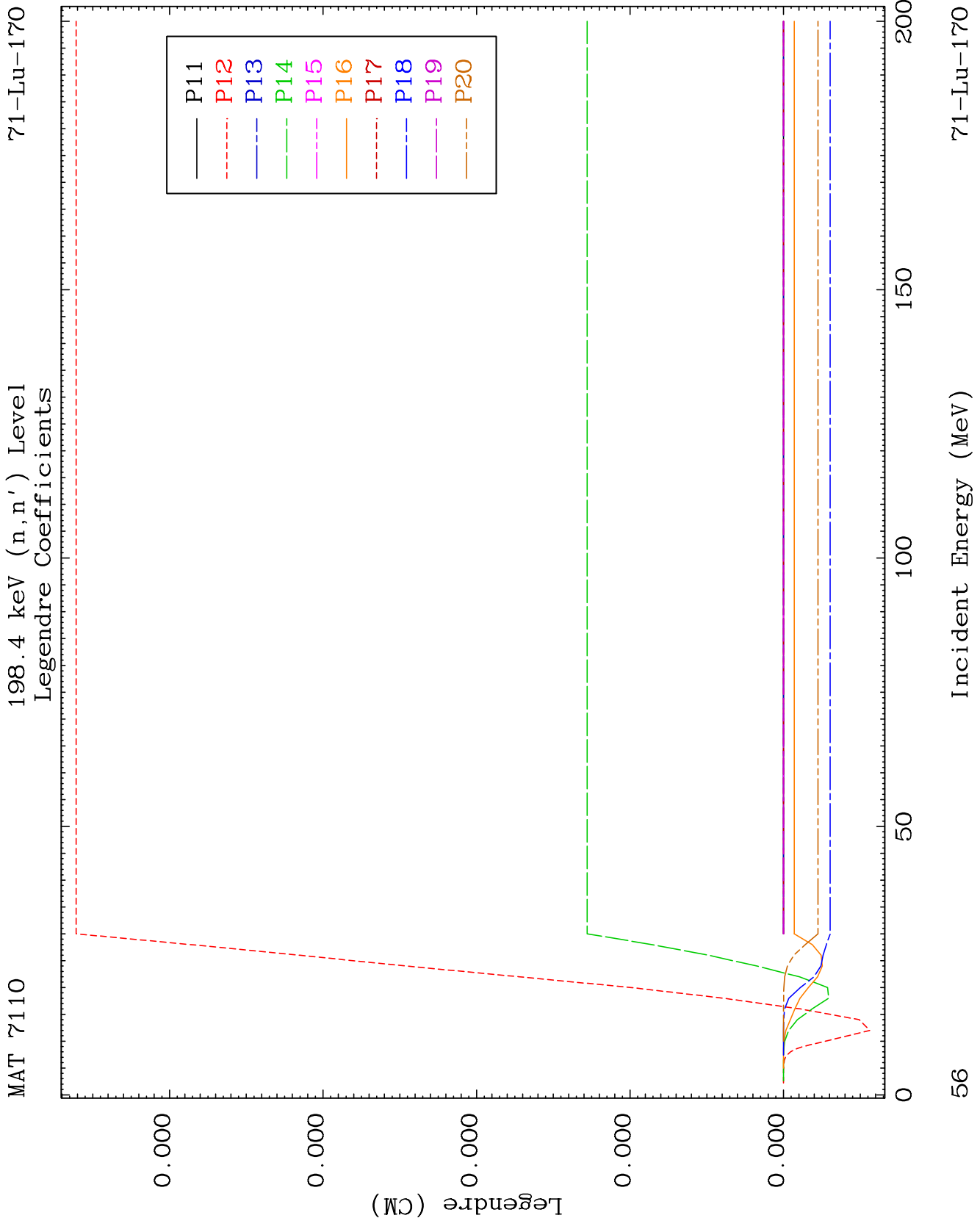


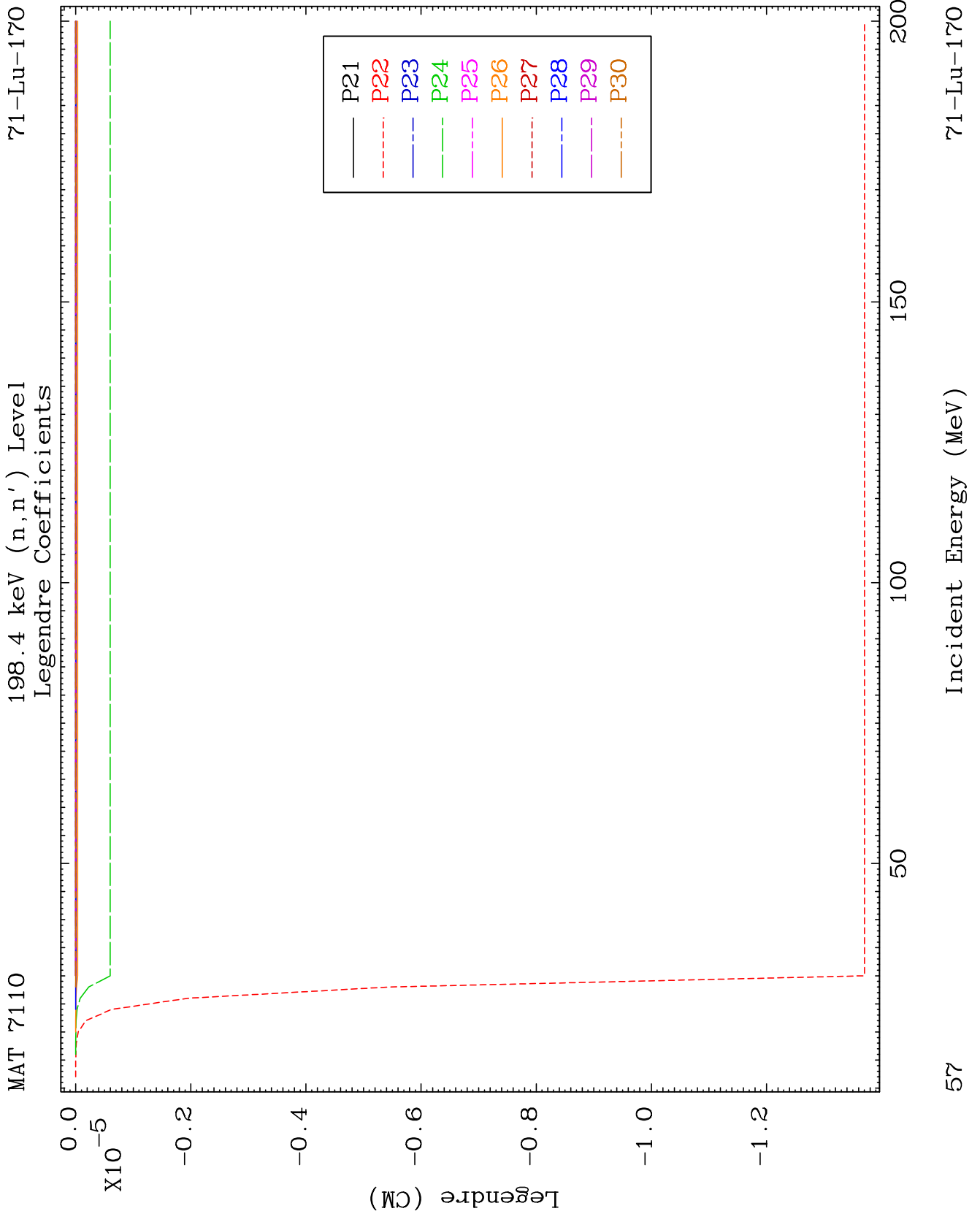


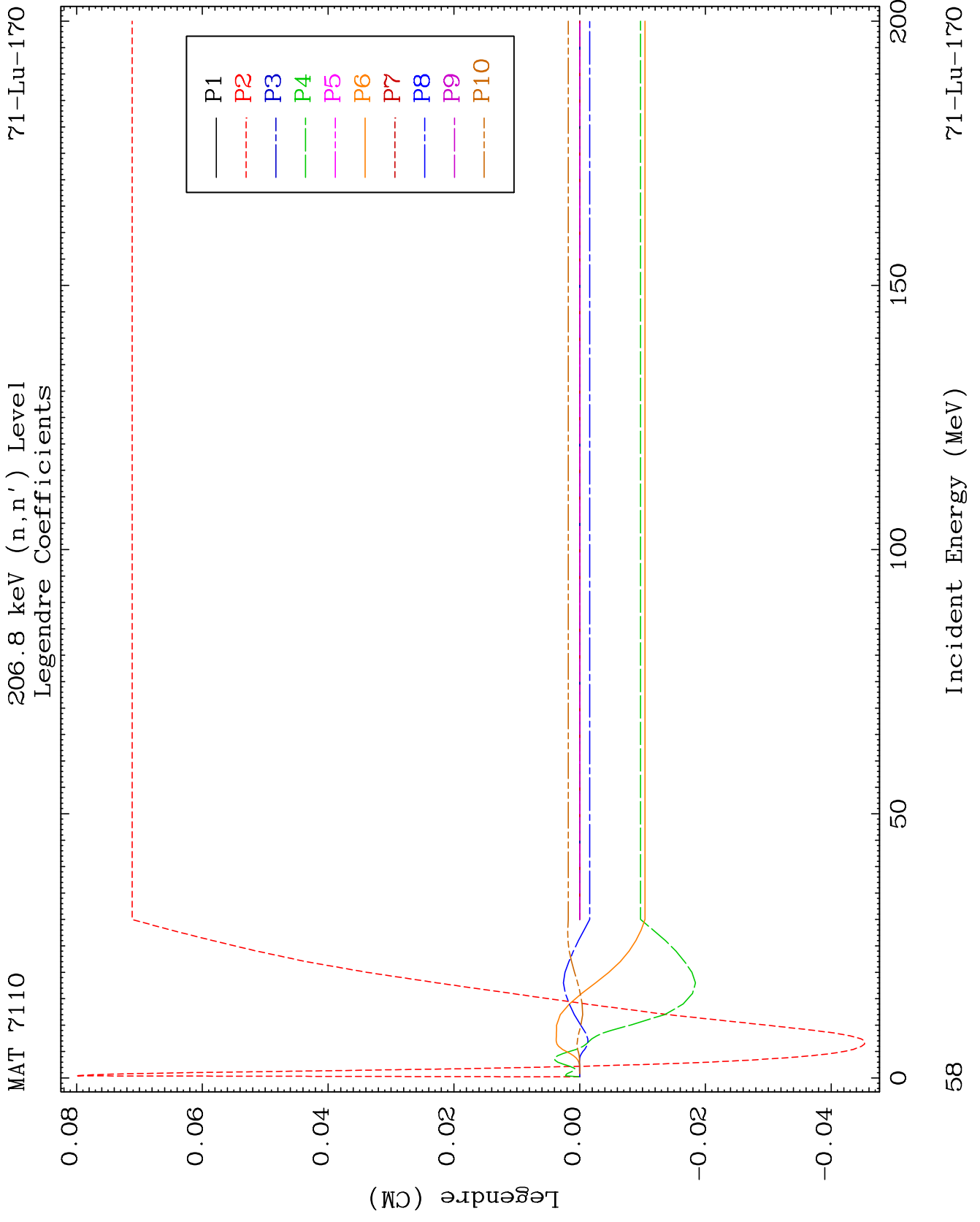


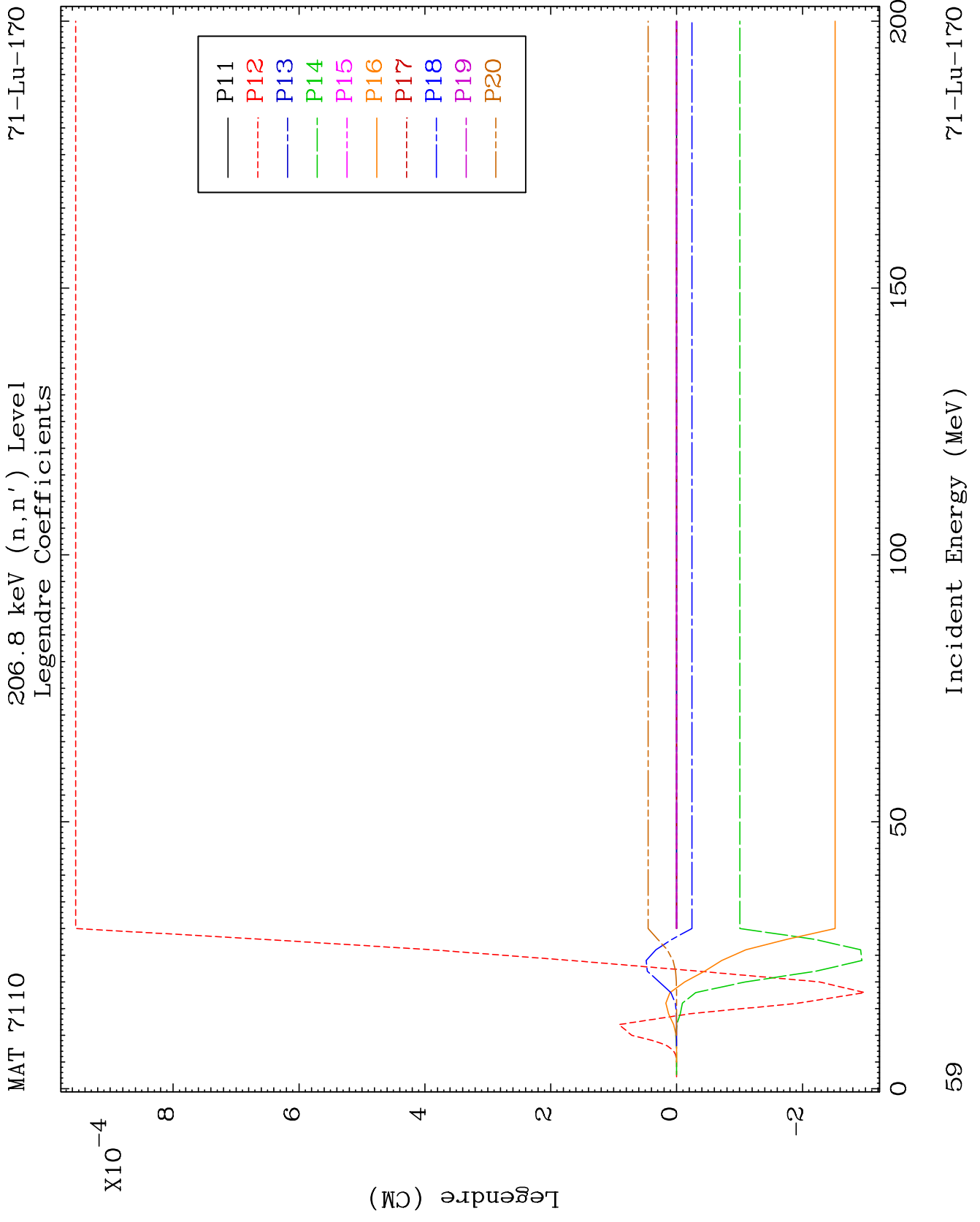


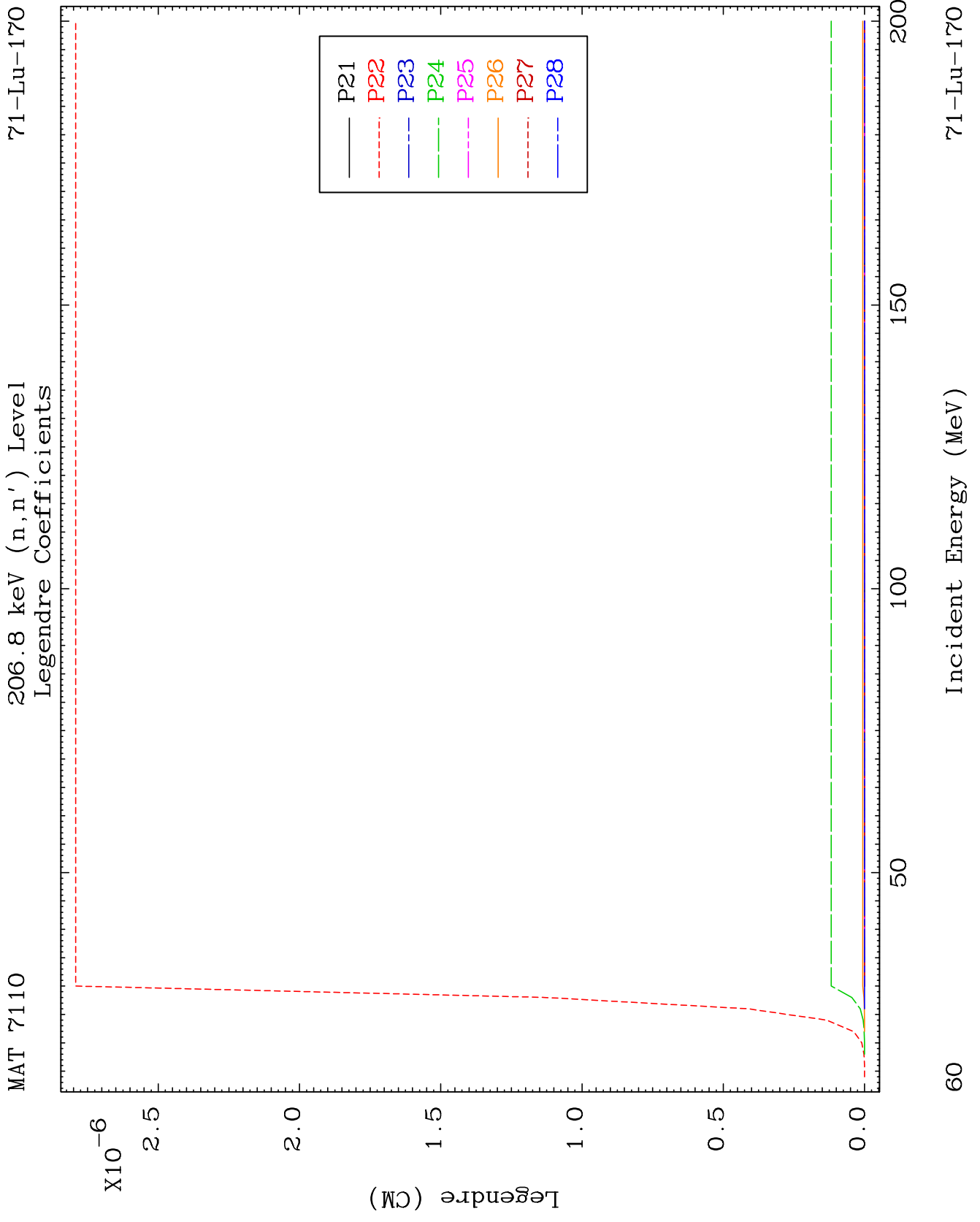


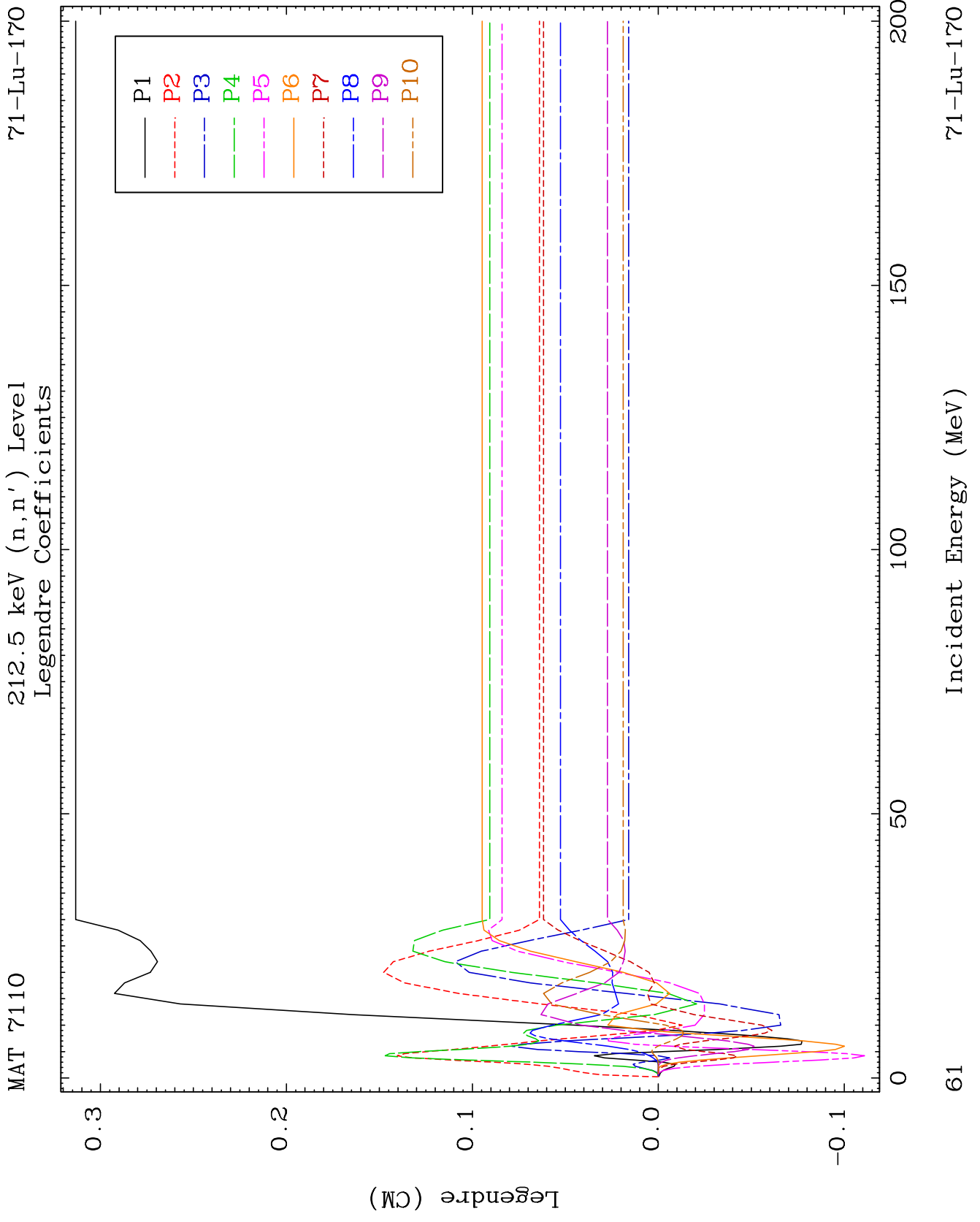


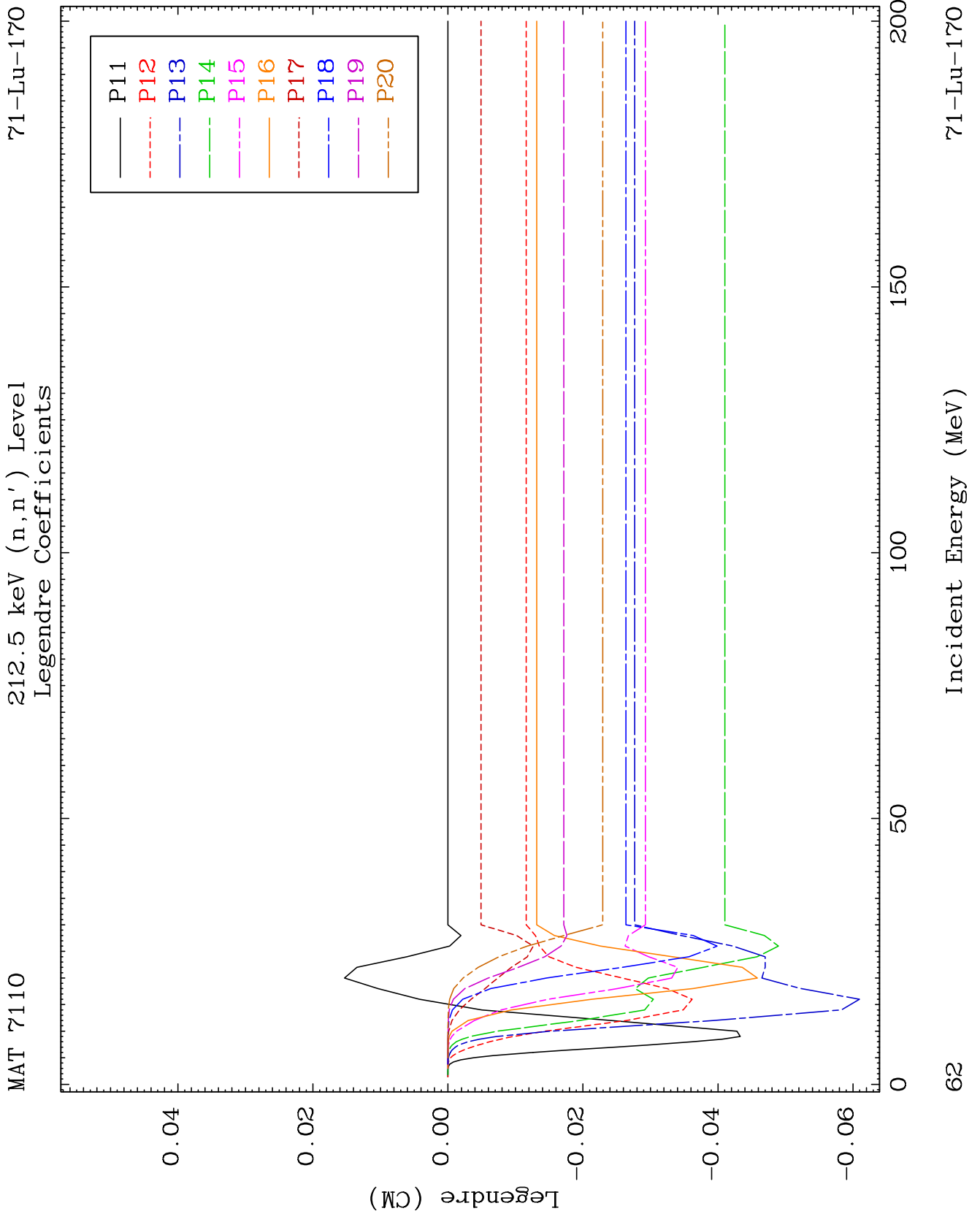


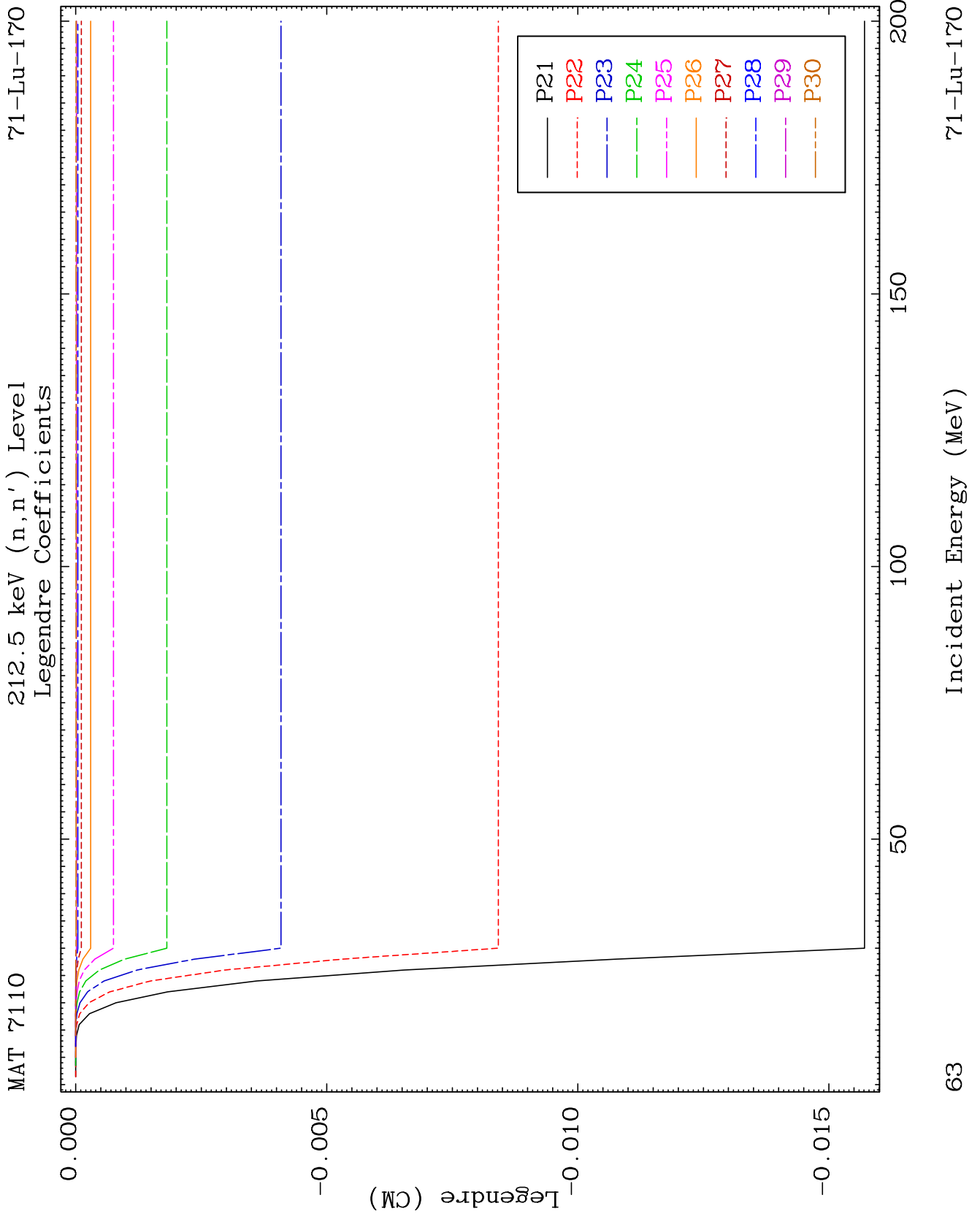




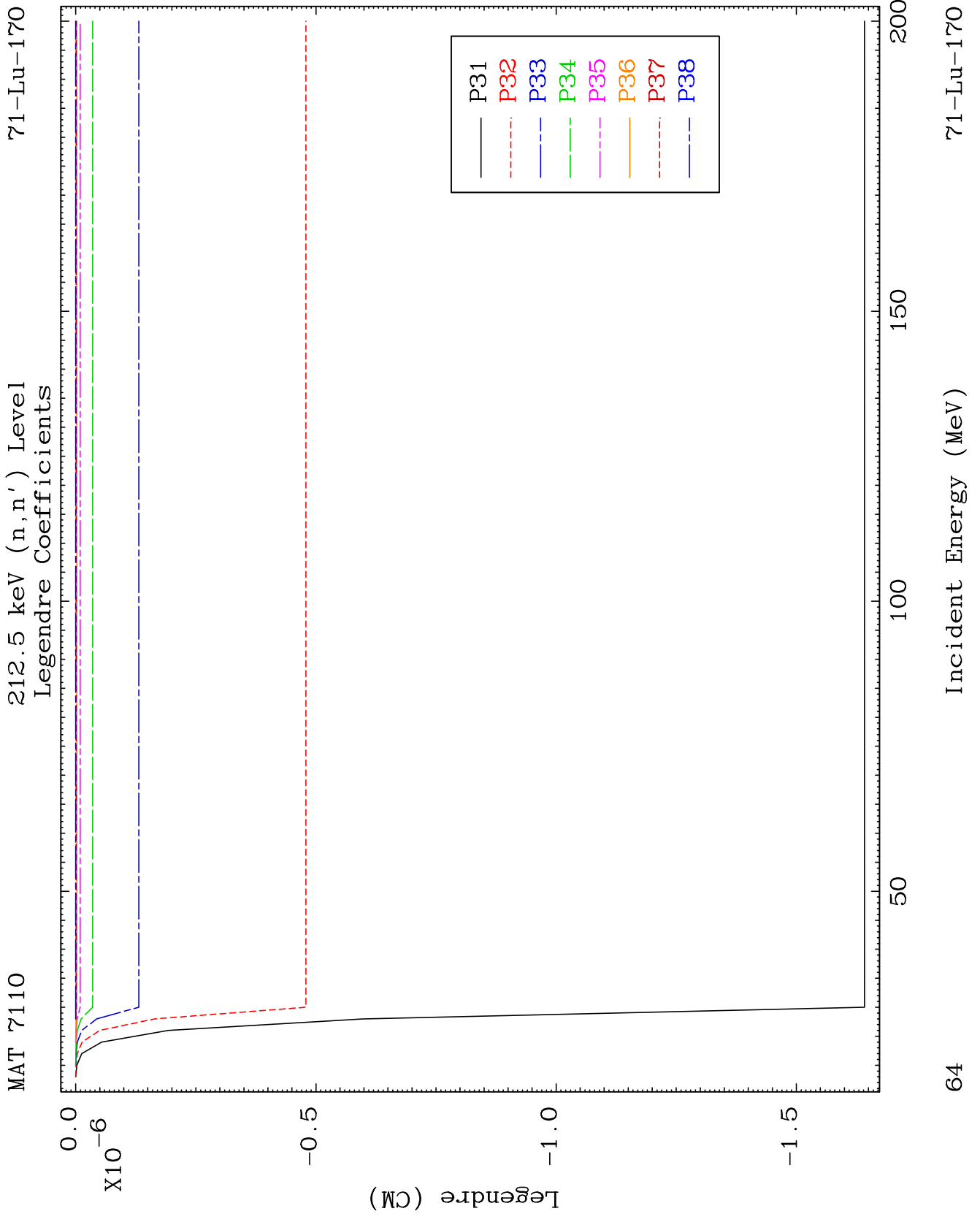


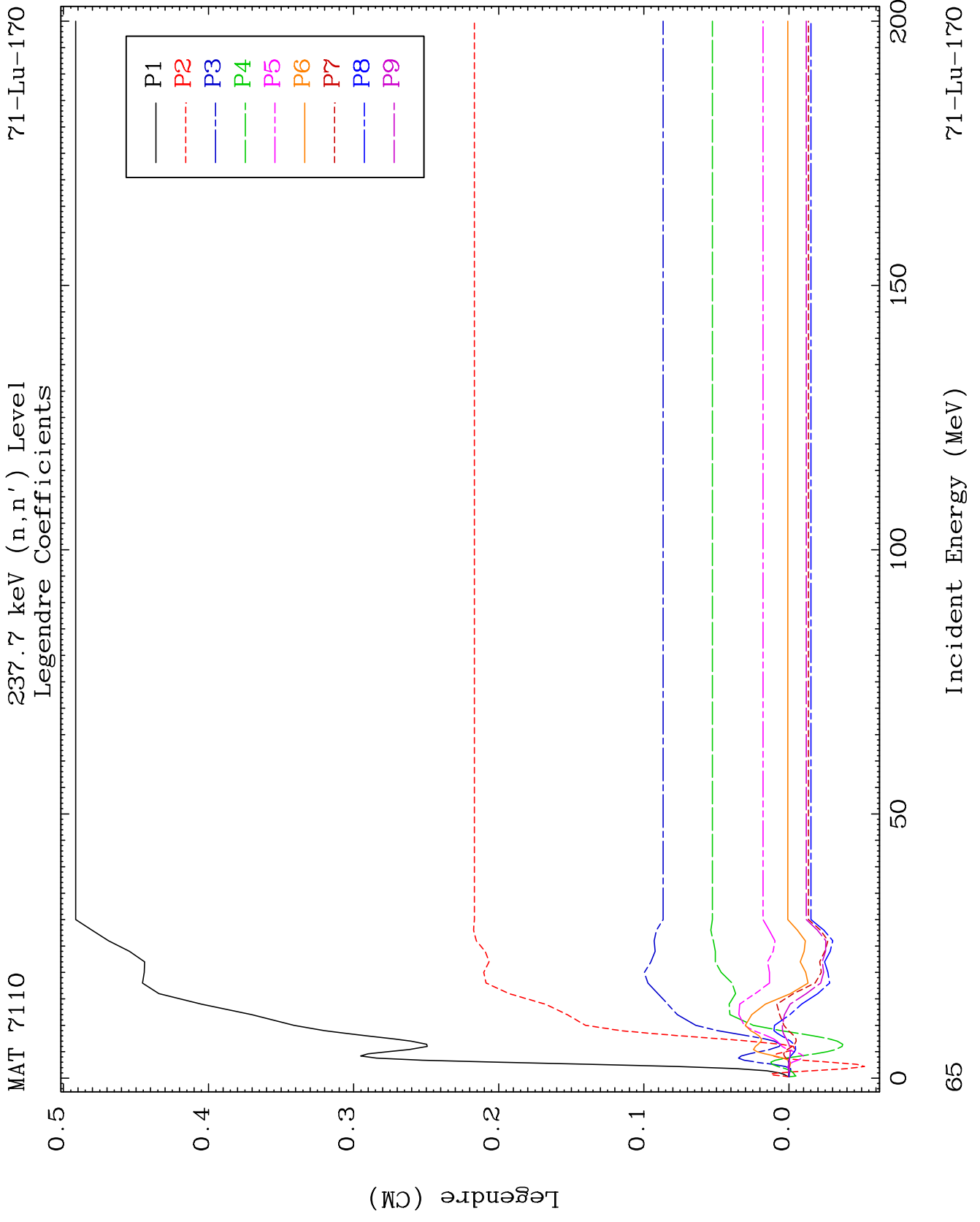








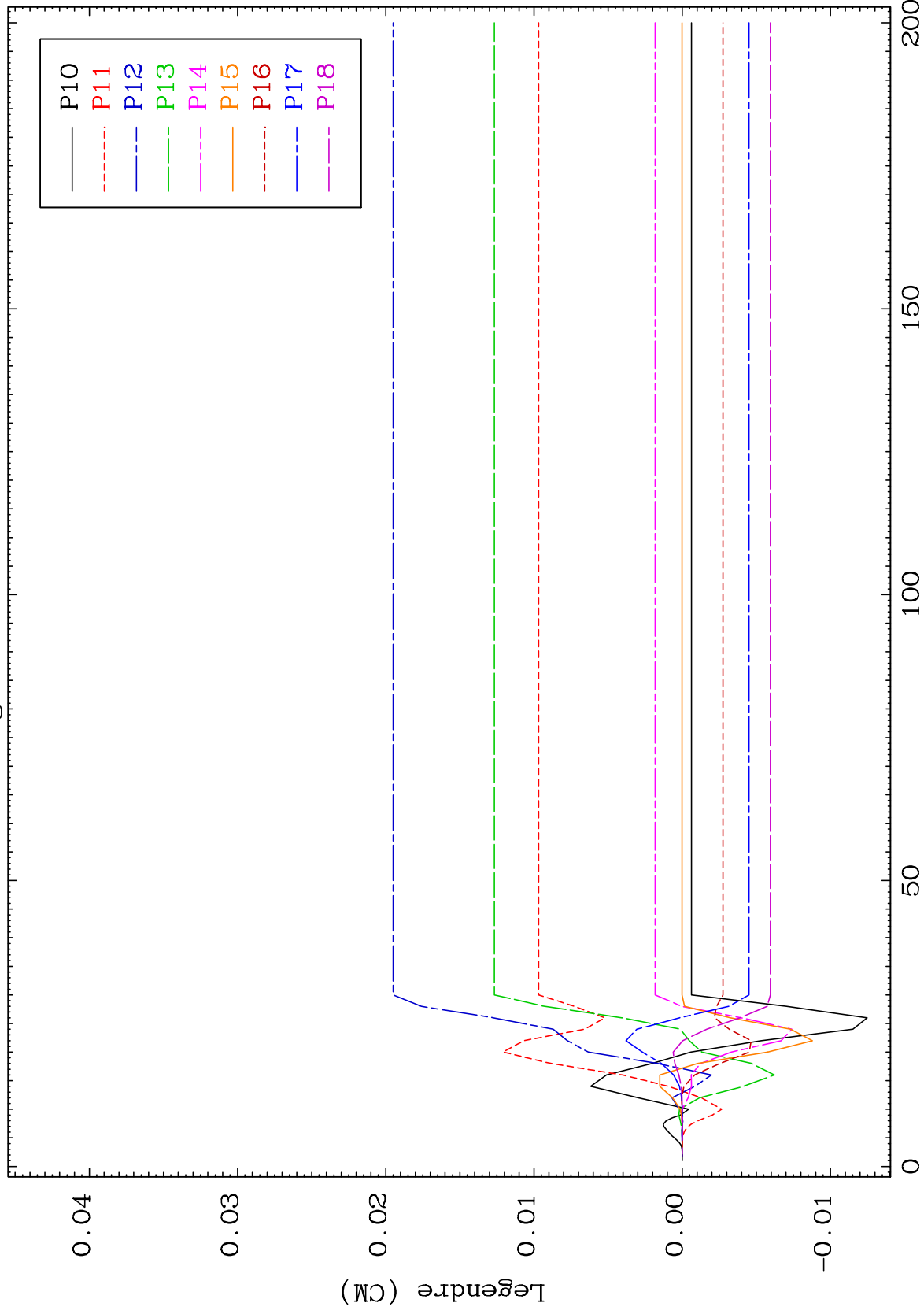




MAT 7110

237.7 keV (n,n') Level  
Legendre Coefficients

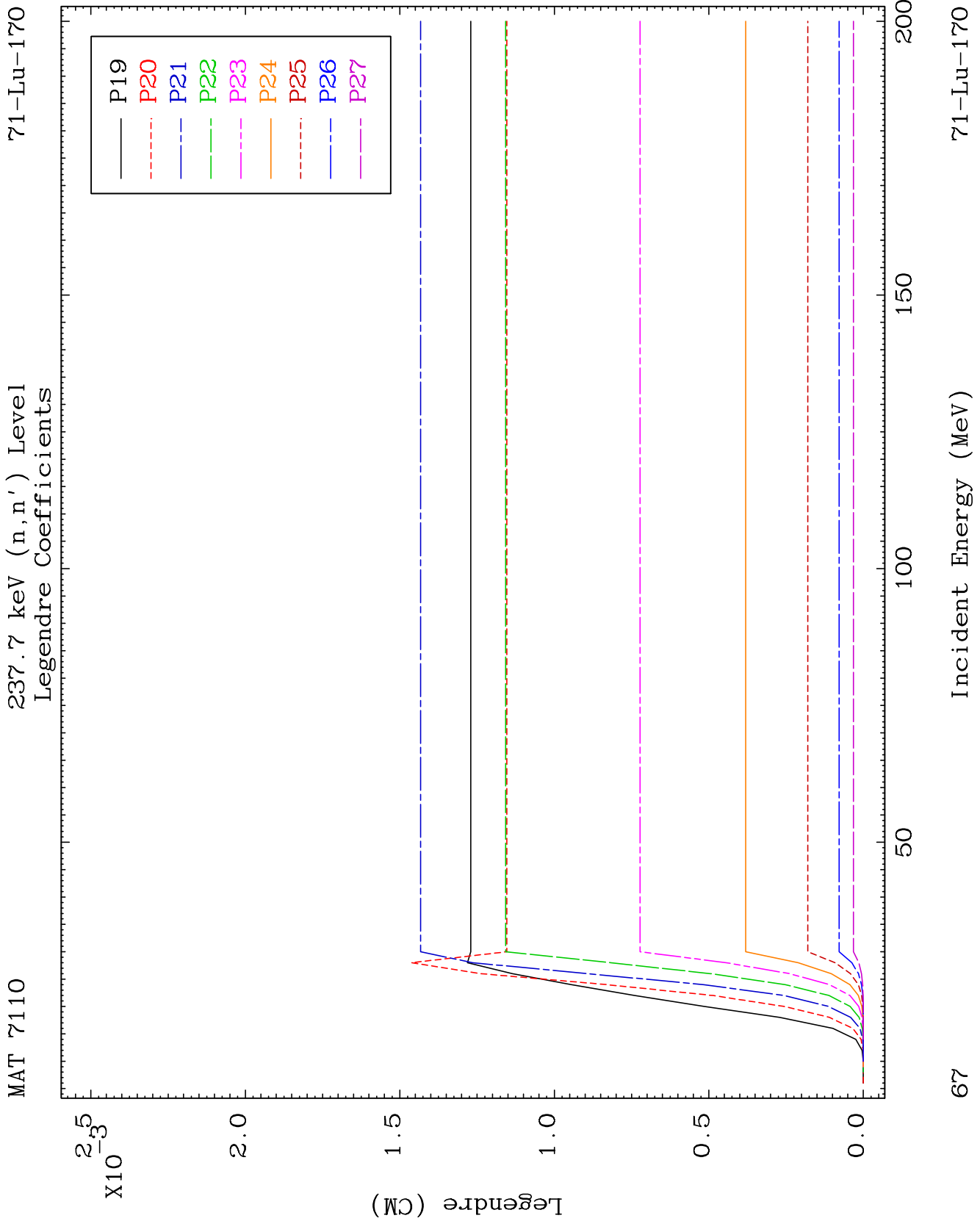
71-Lu-170

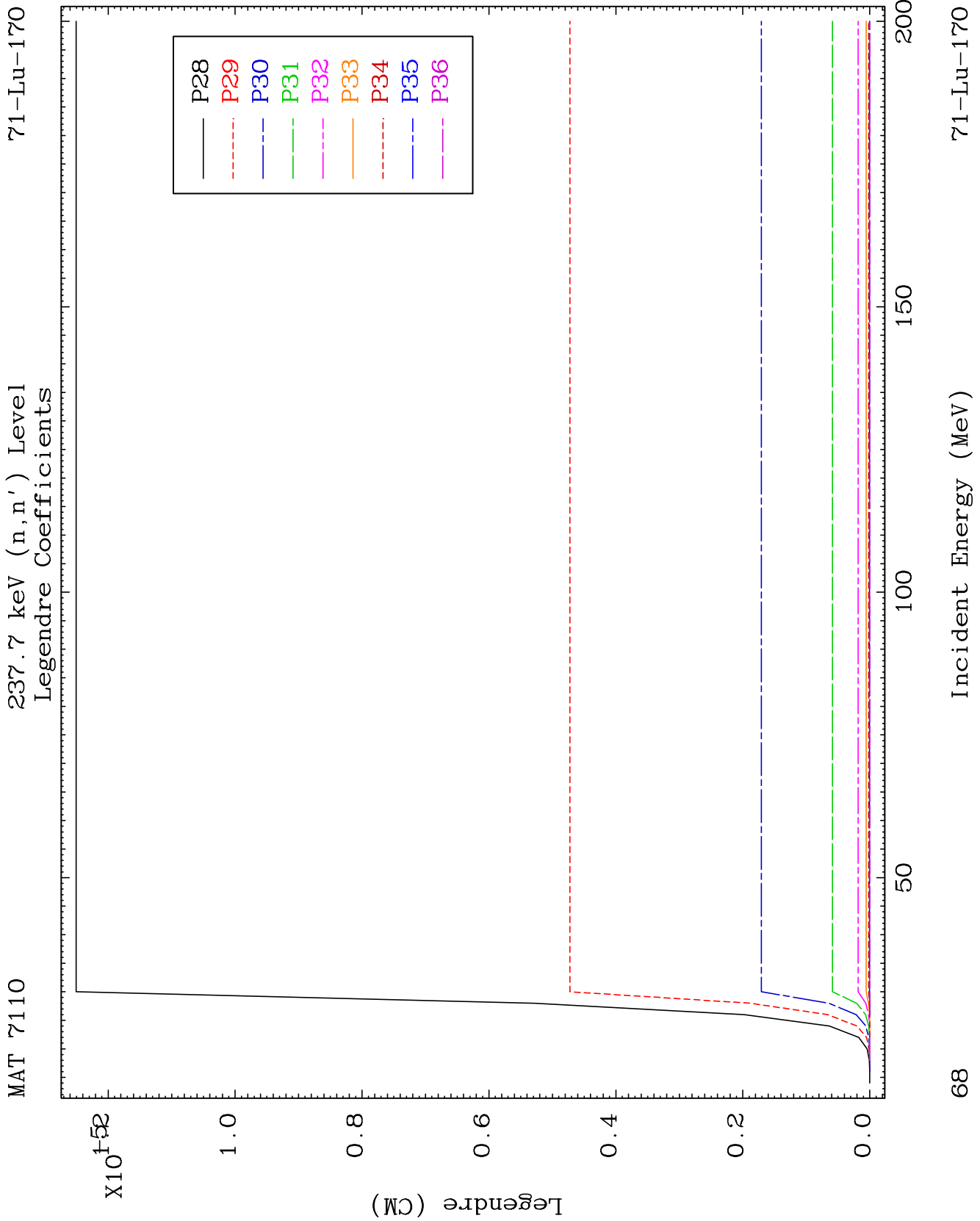


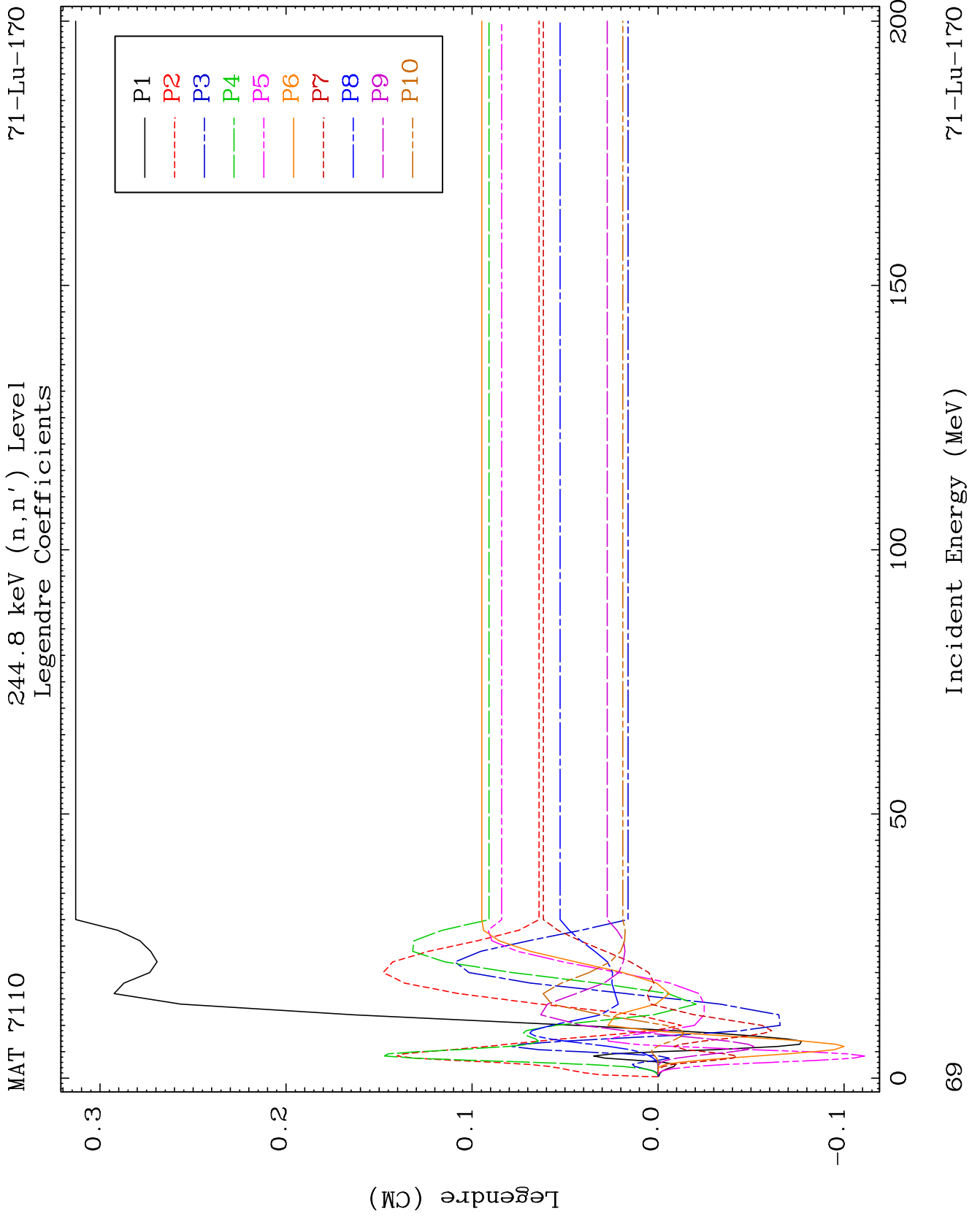
66

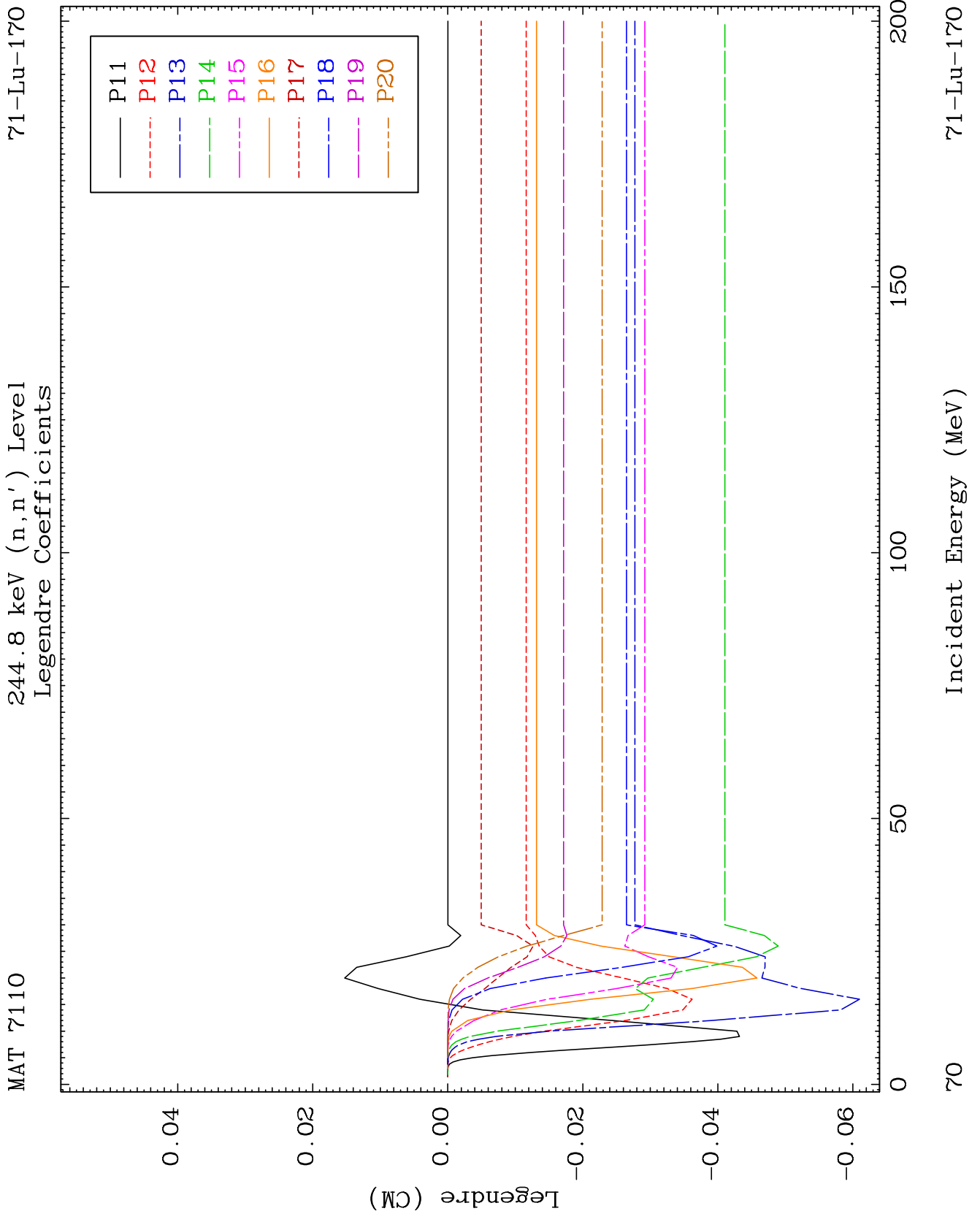
Incident Energy (MeV)

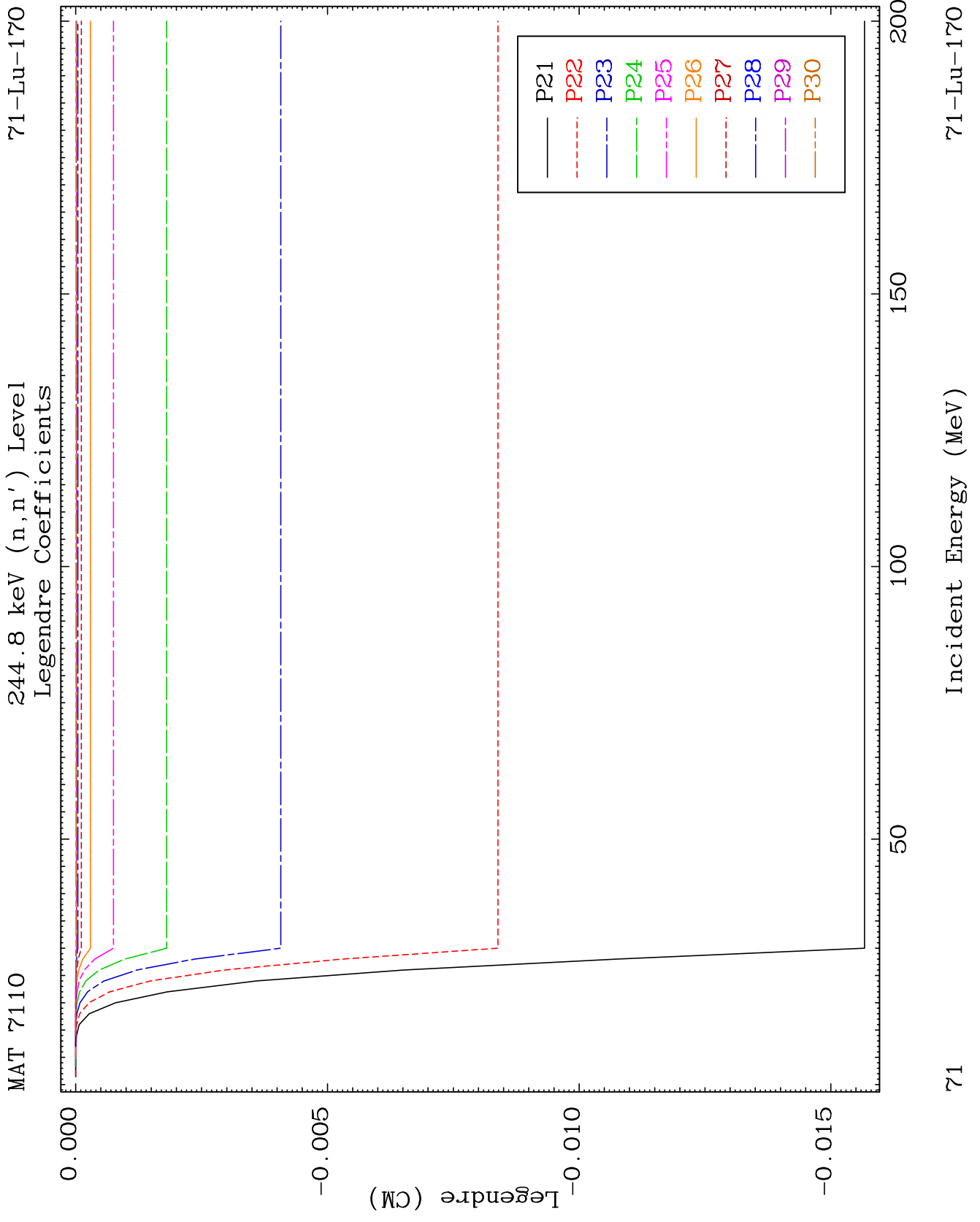
71-Lu-170





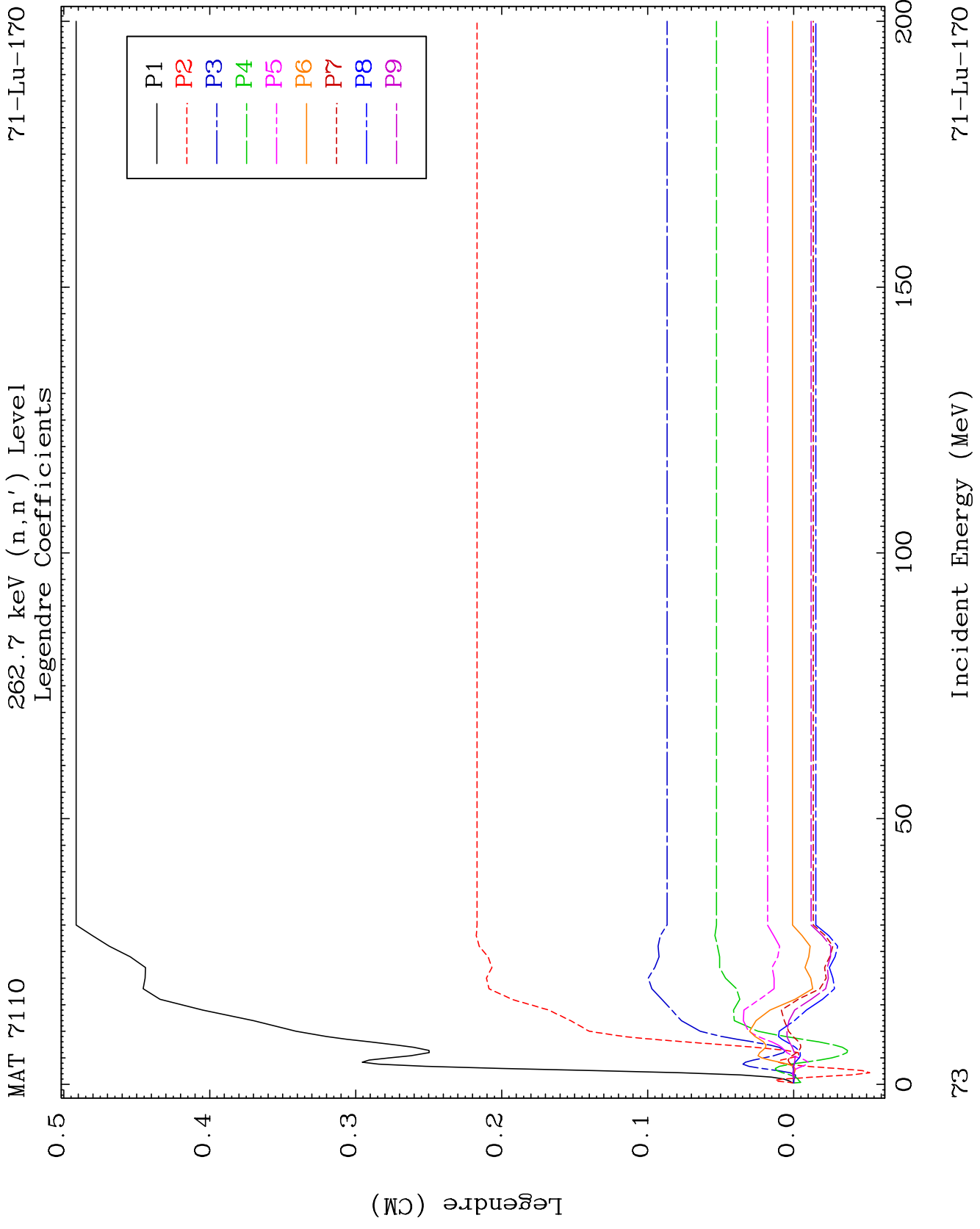


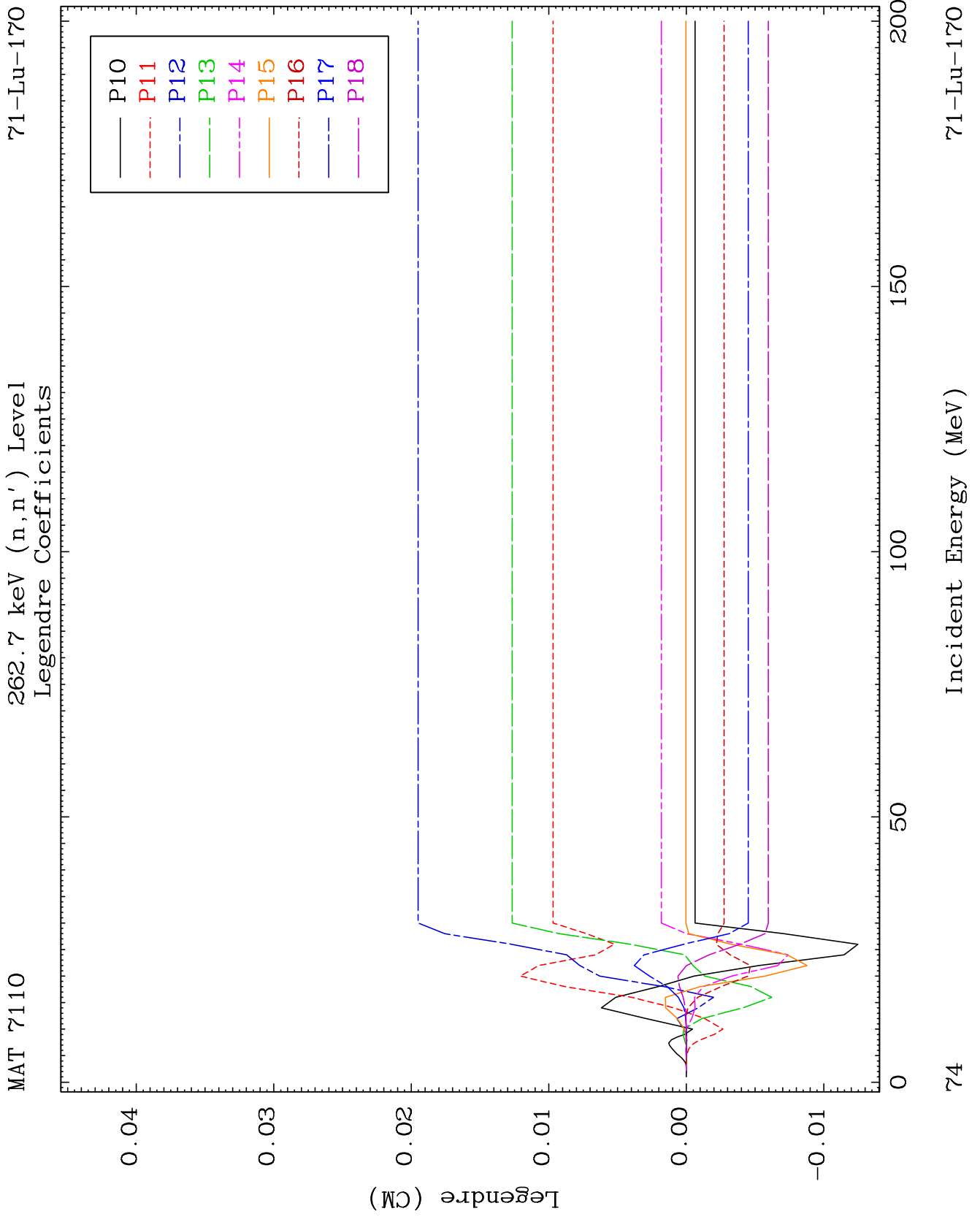


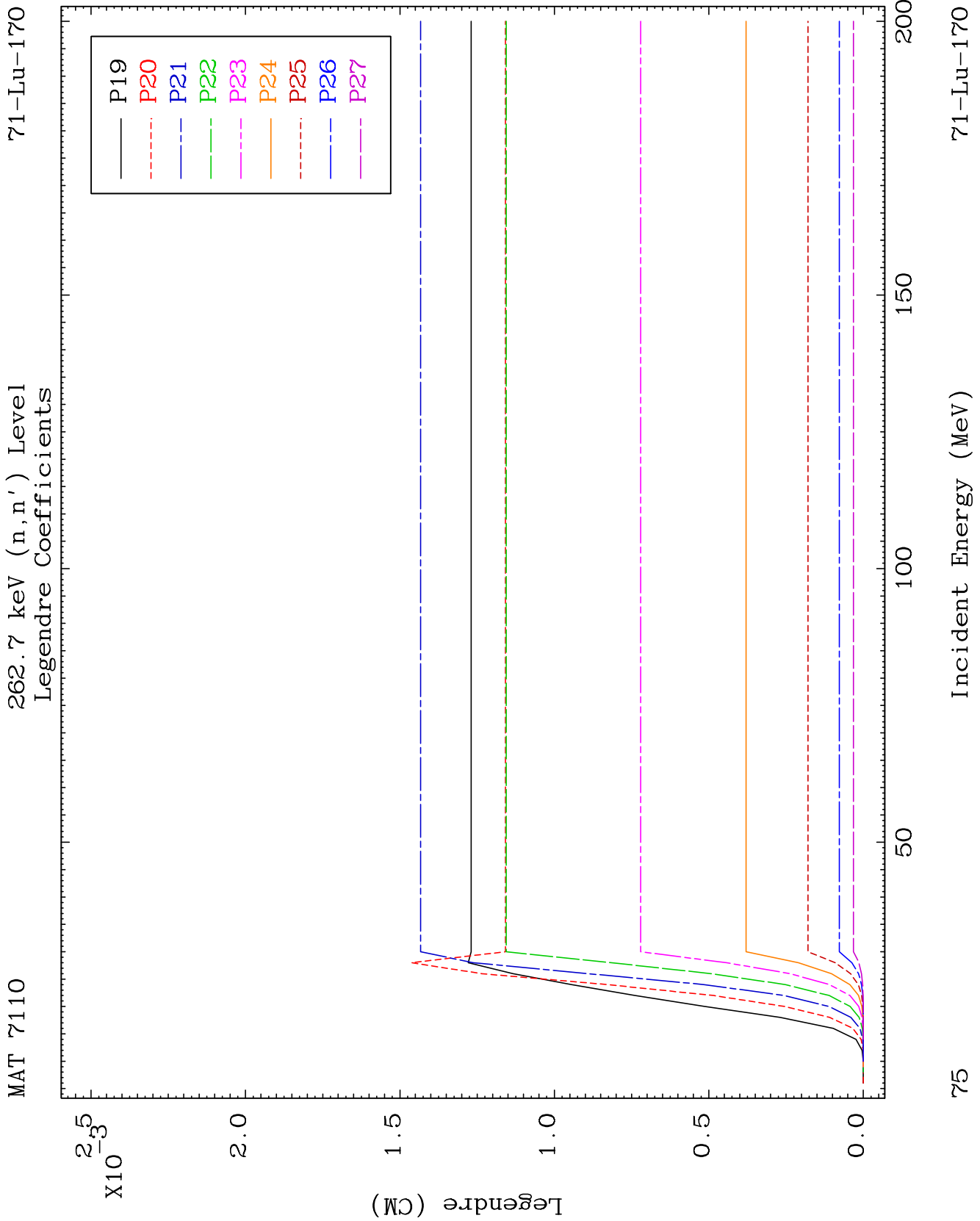


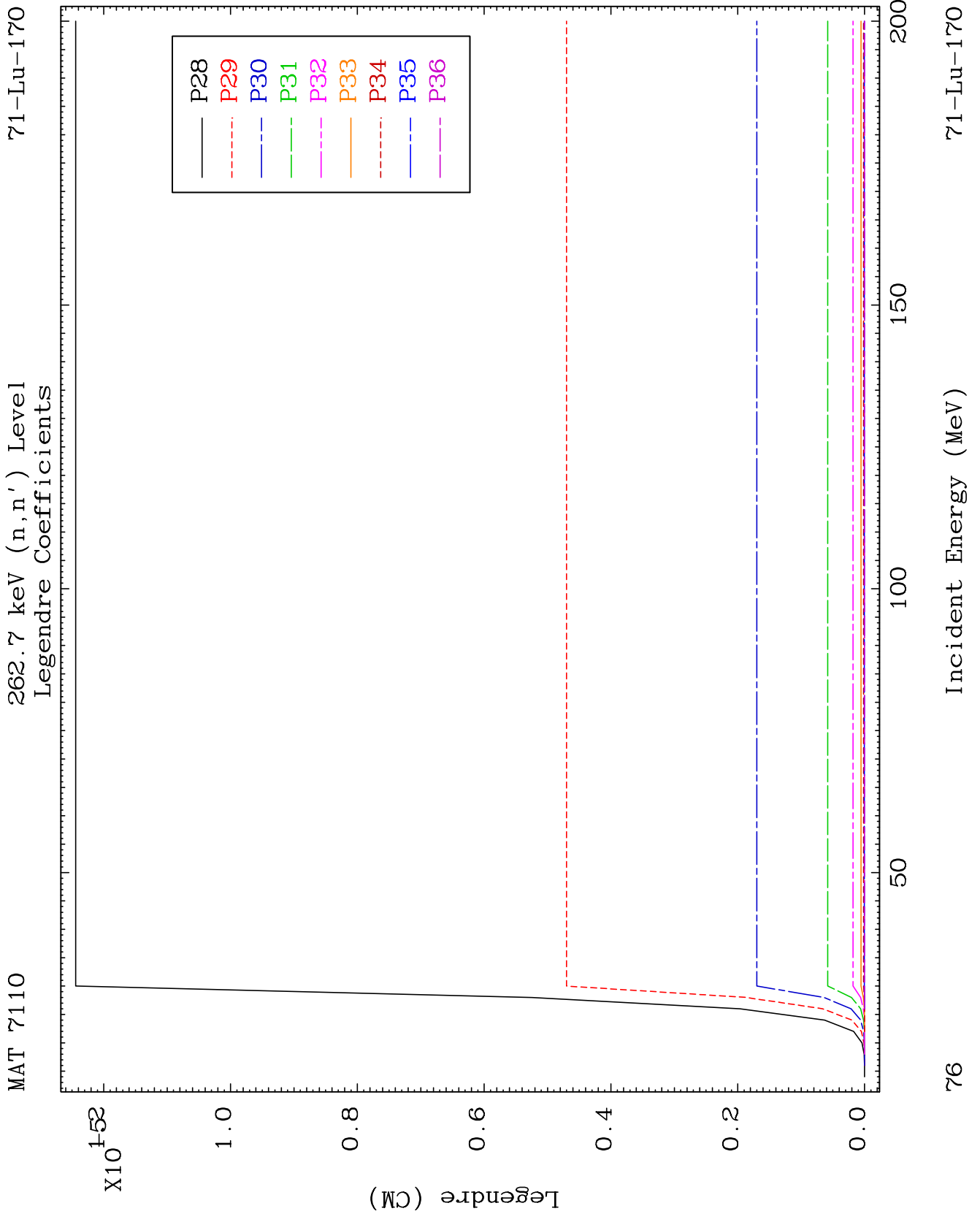










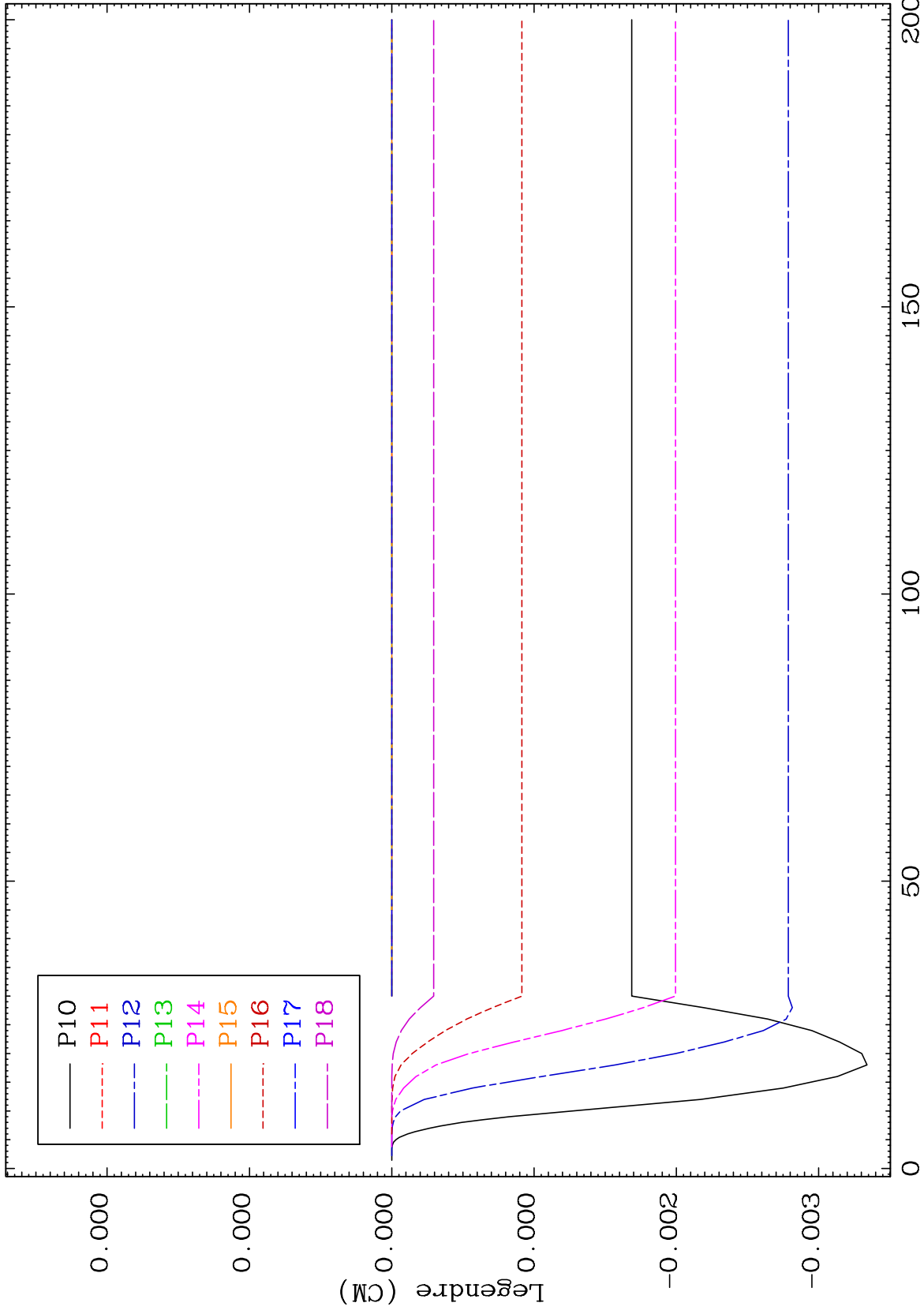




MAT 7110

283.9 keV (n,n') Level  
Legendre Coefficients

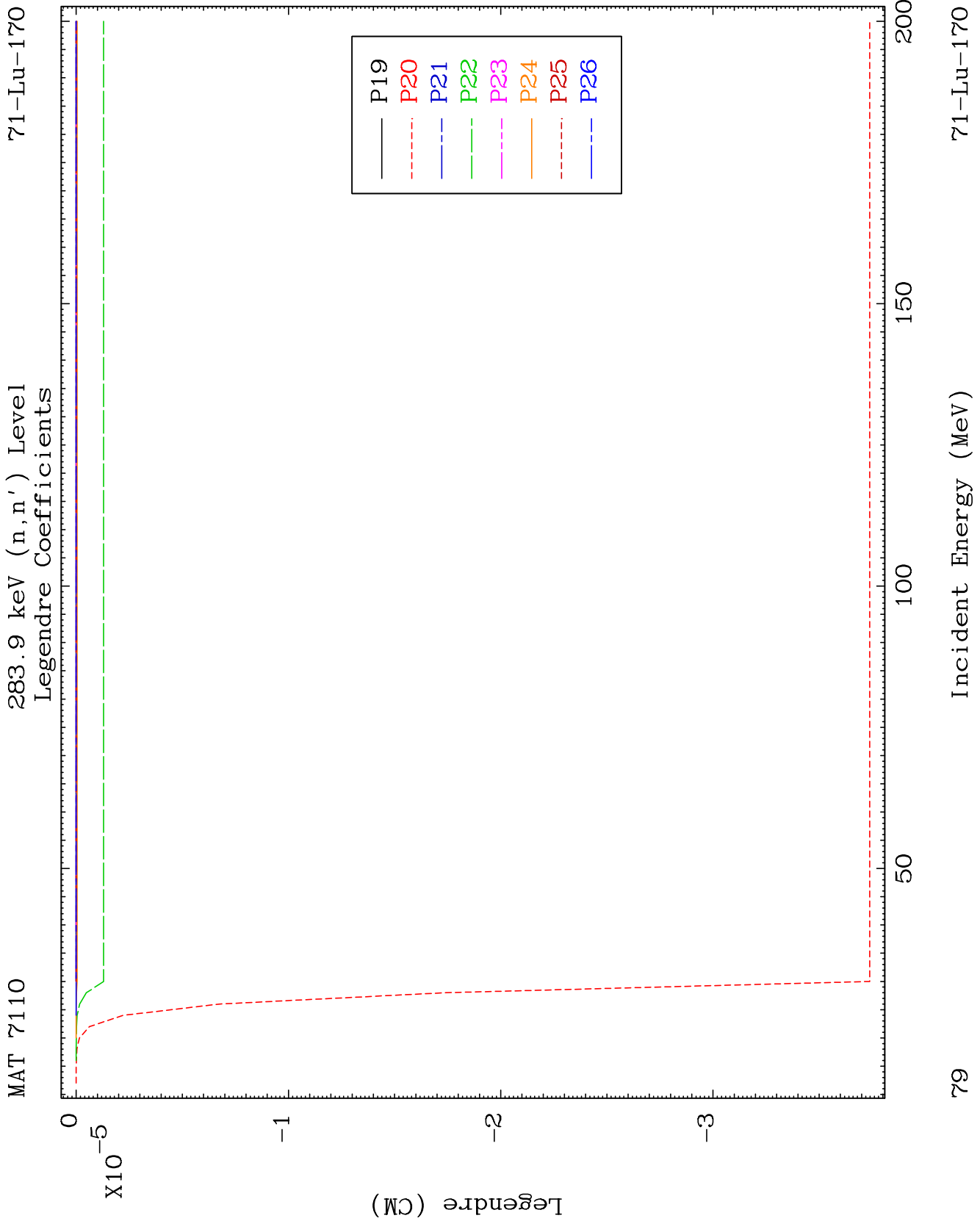
71-Lu-170



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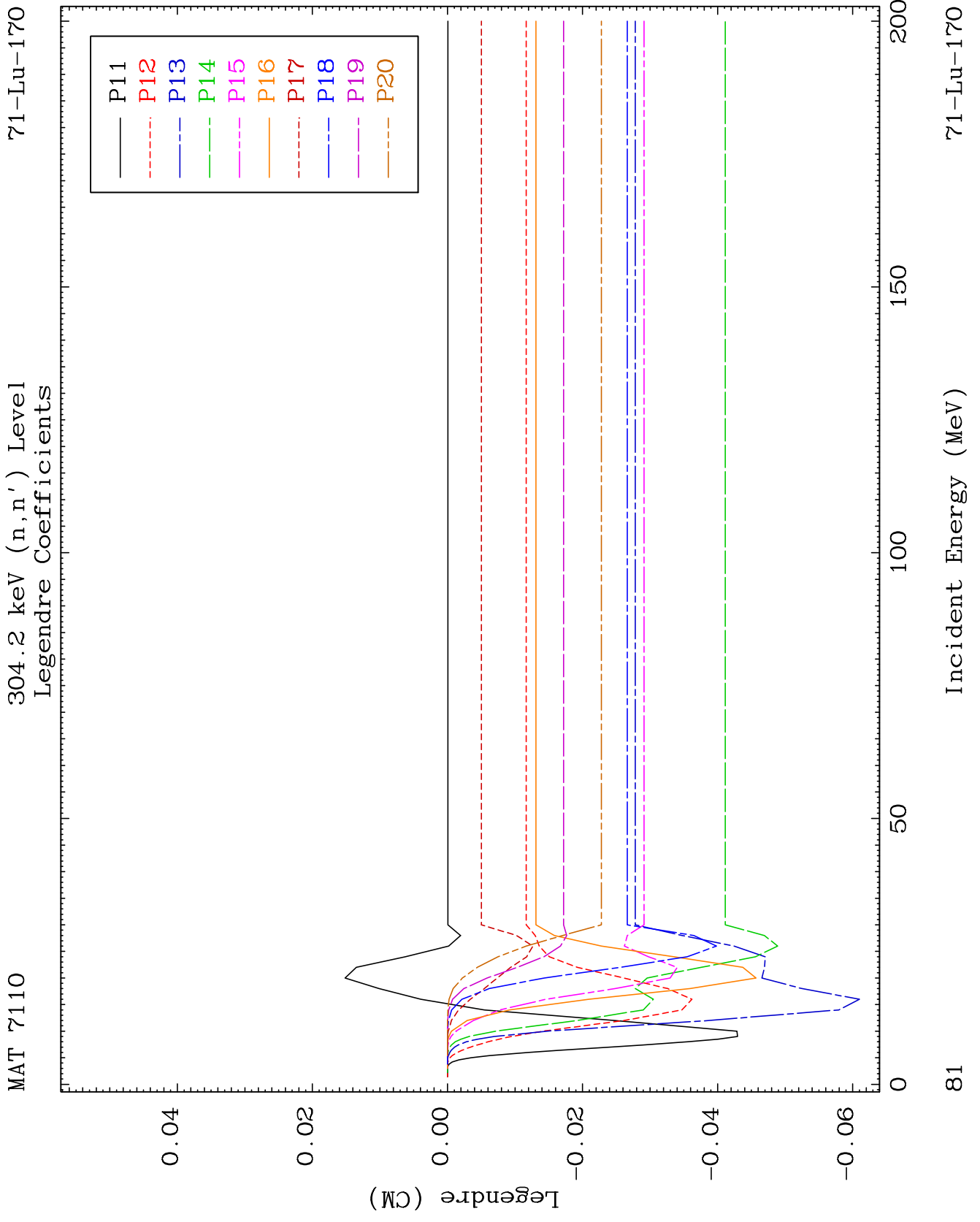
71-Lu-170

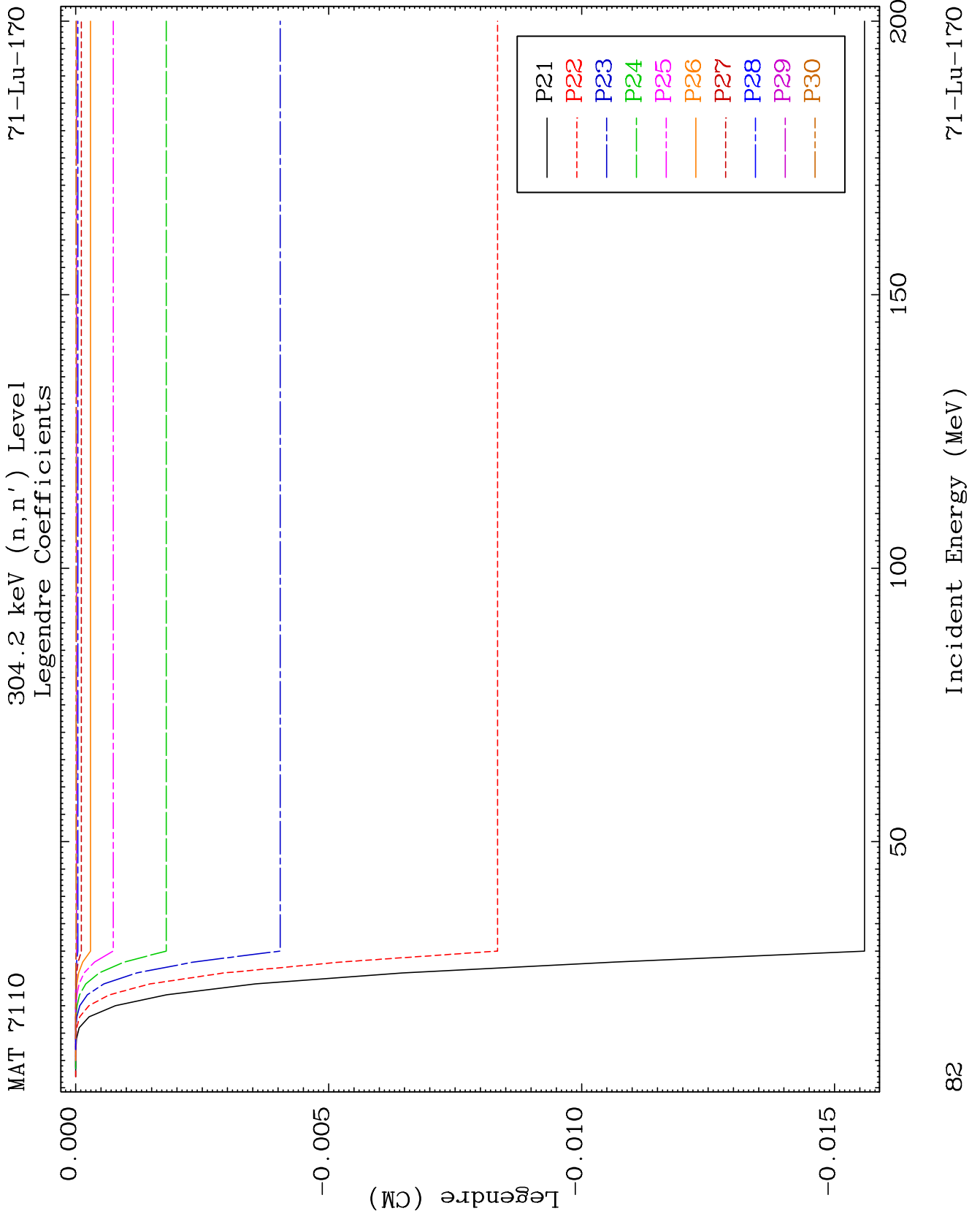
71-Lu-170

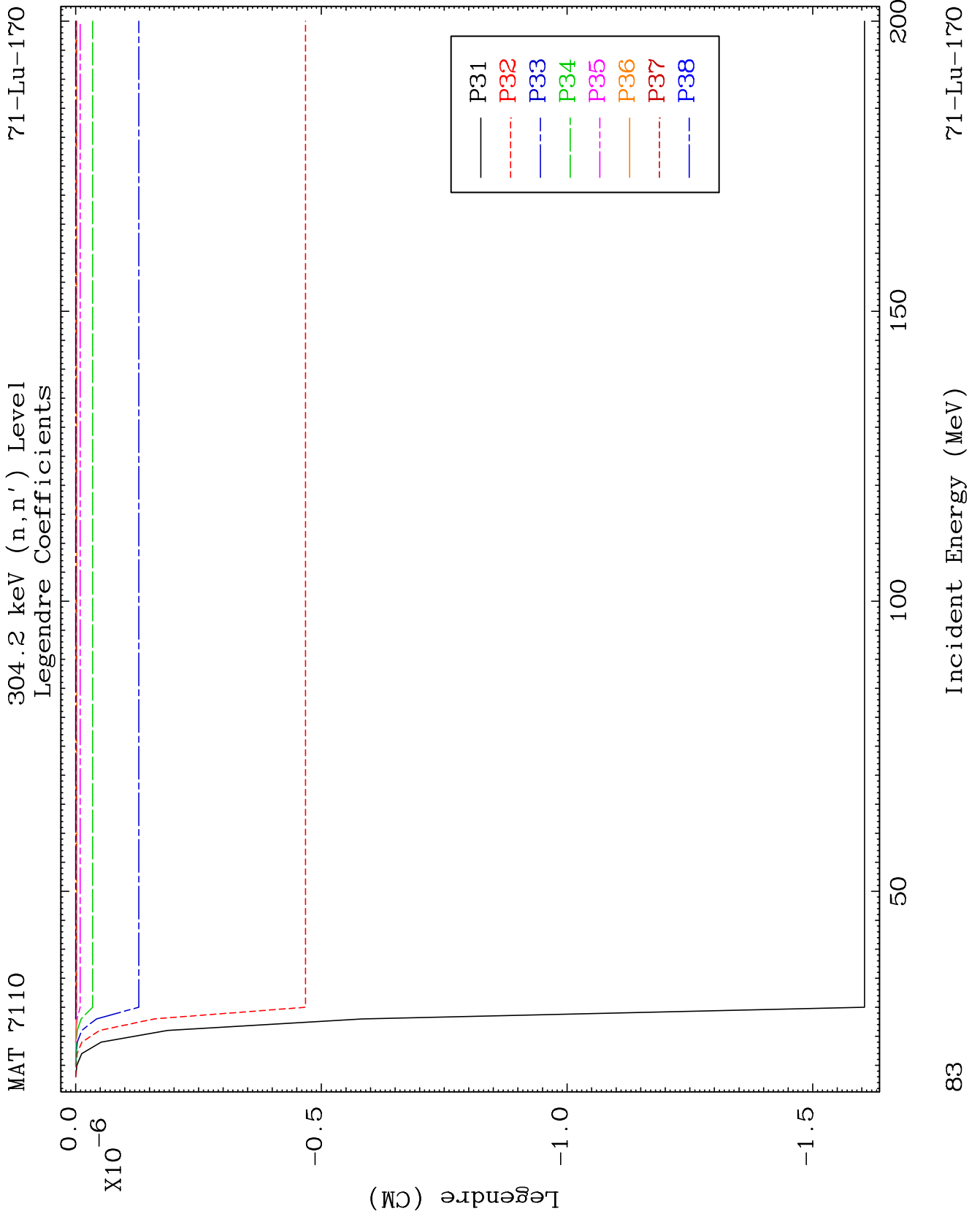




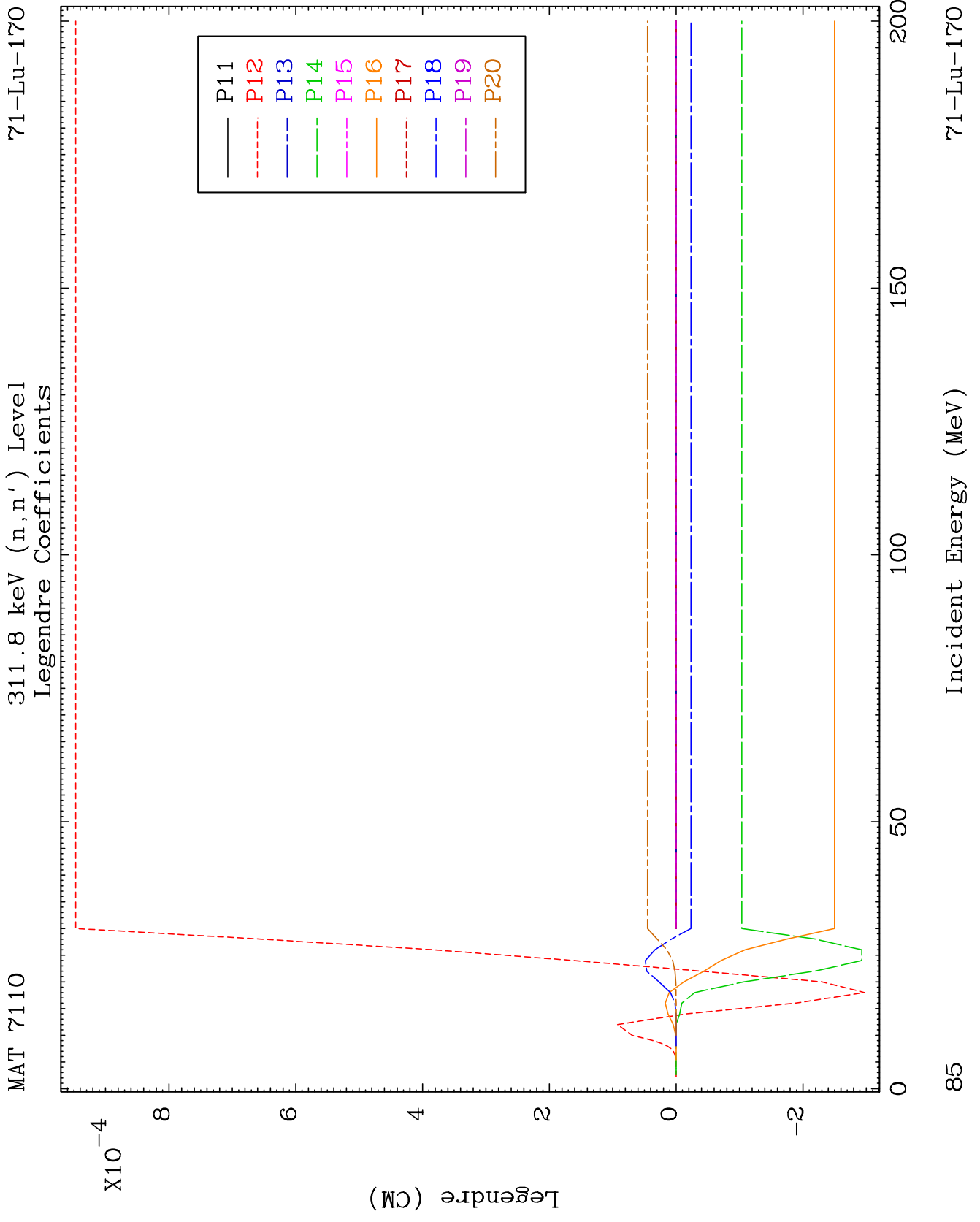




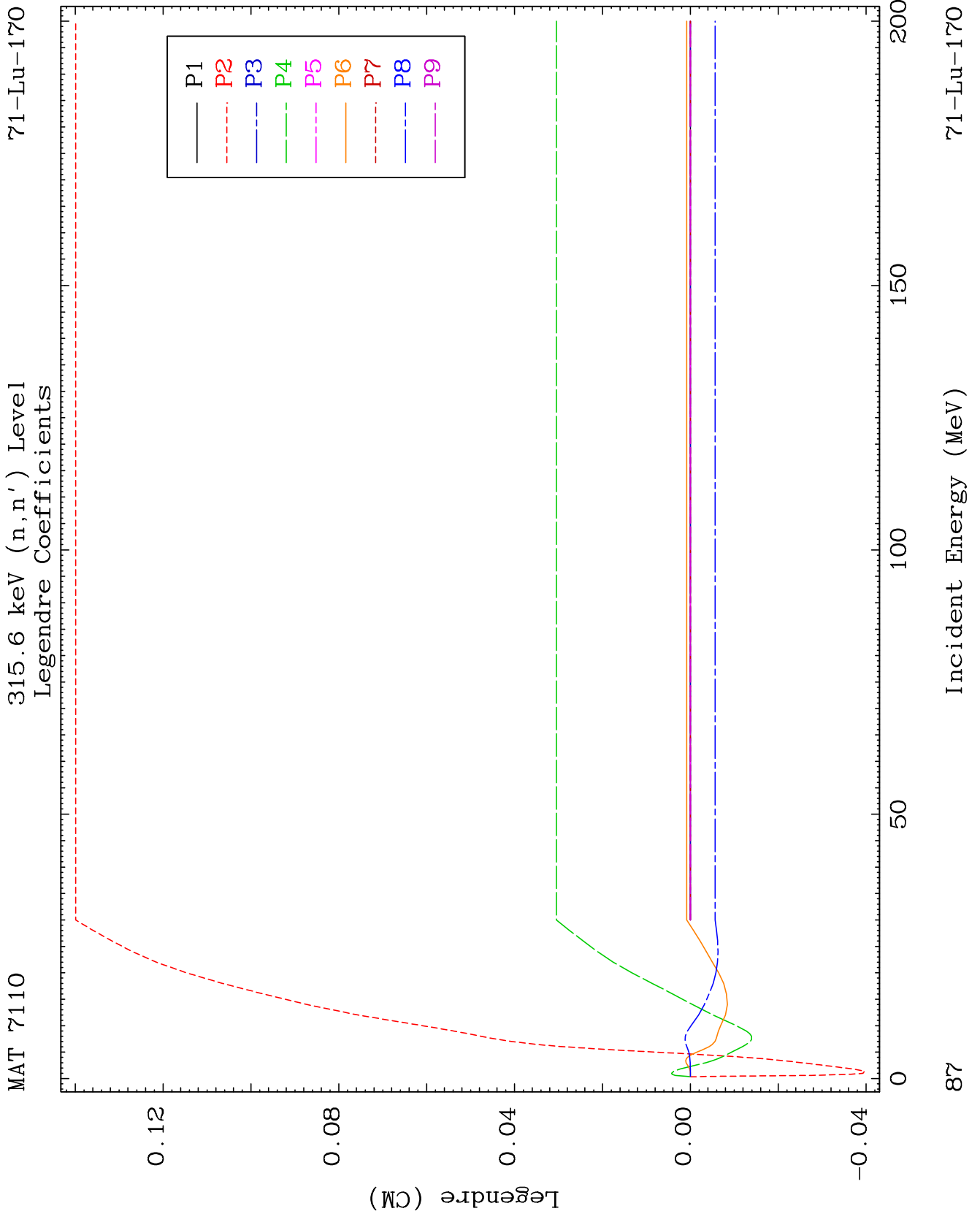




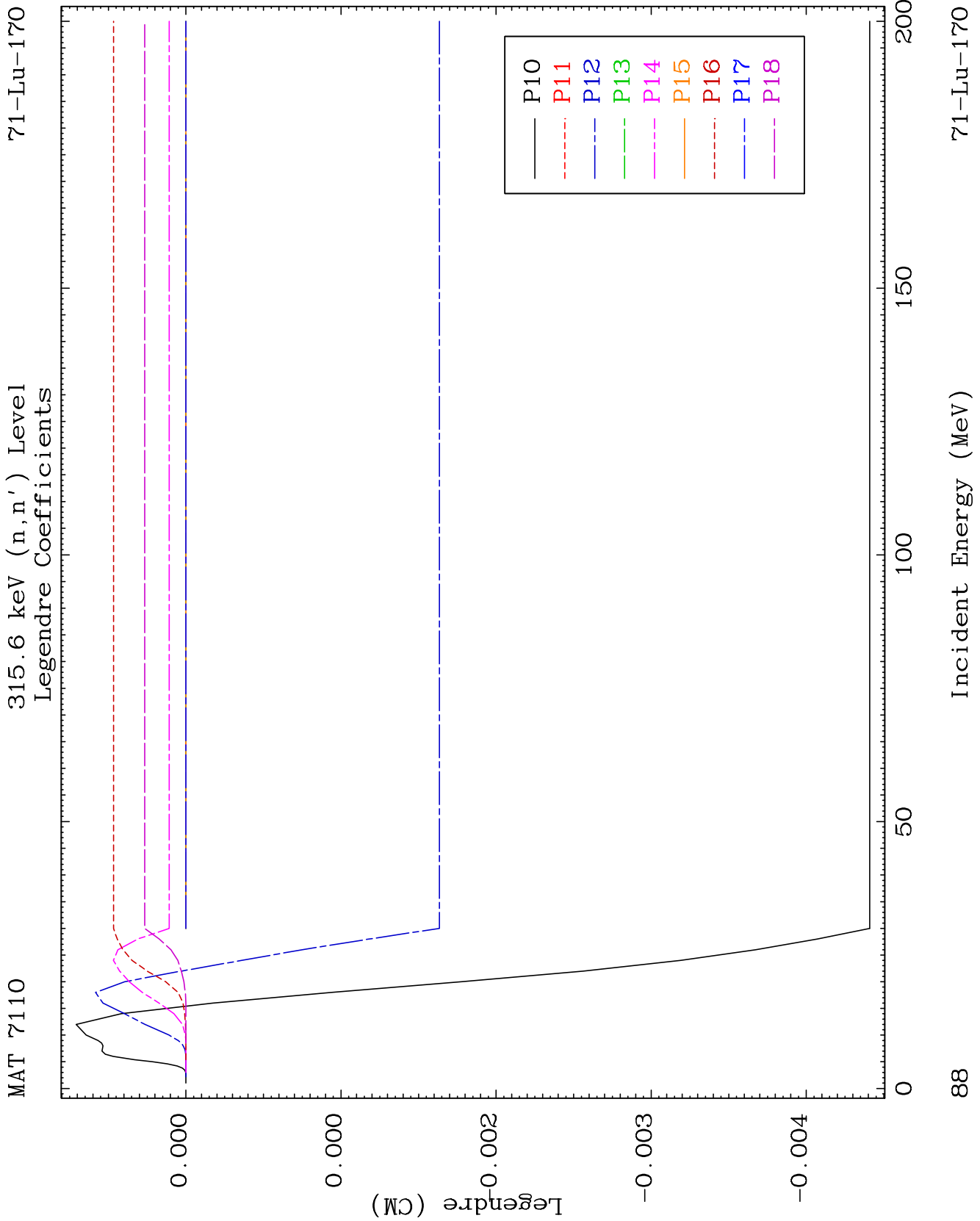




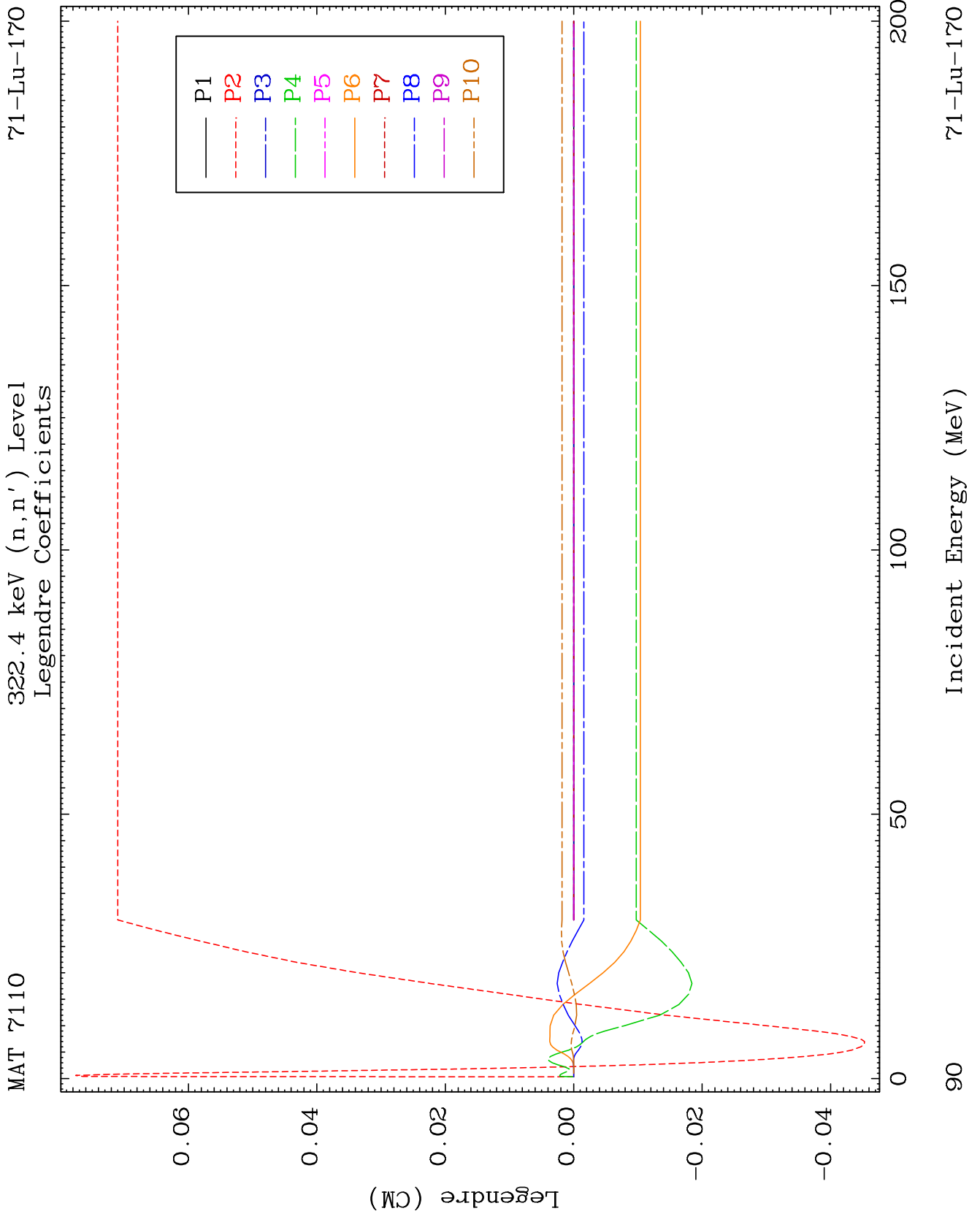


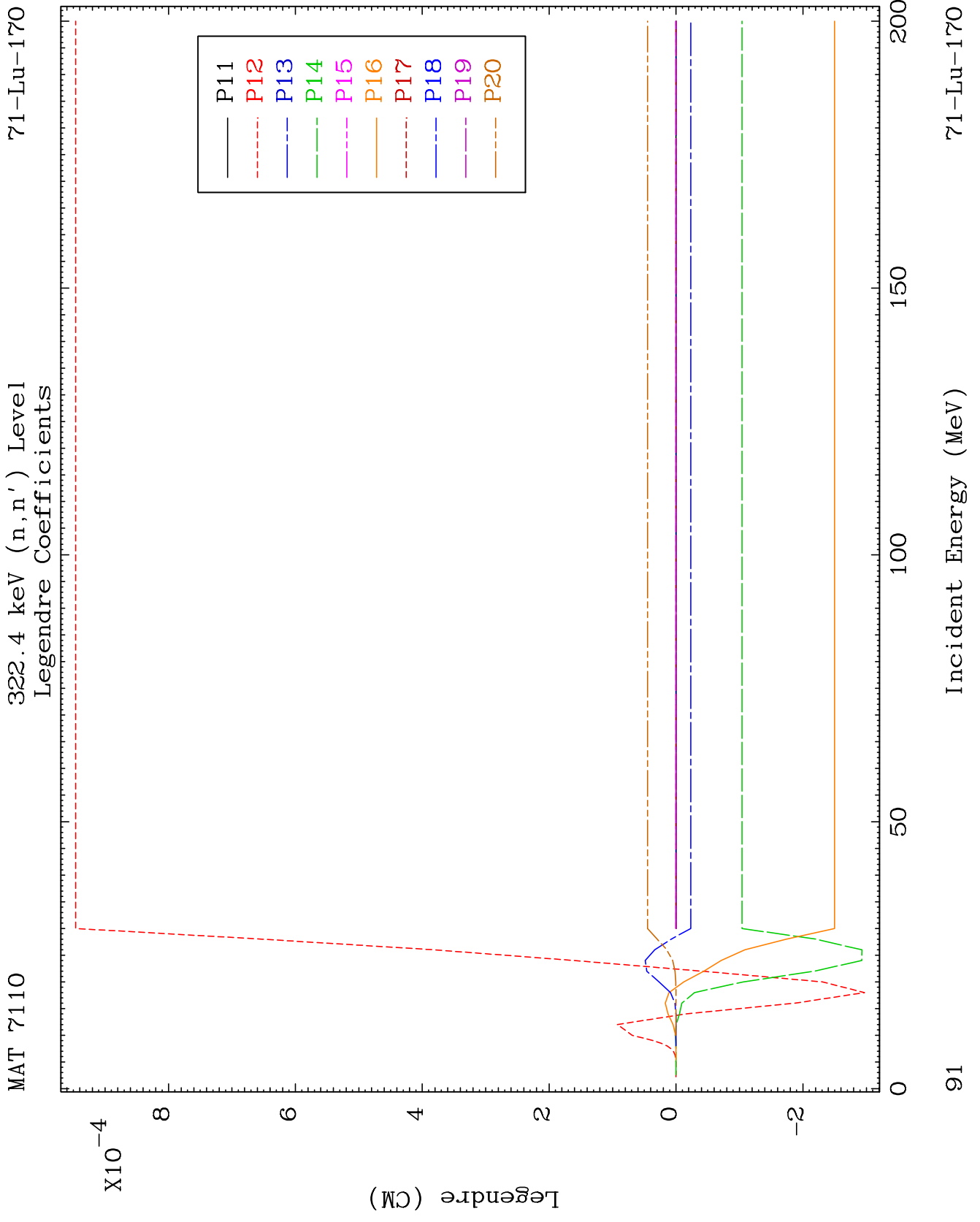


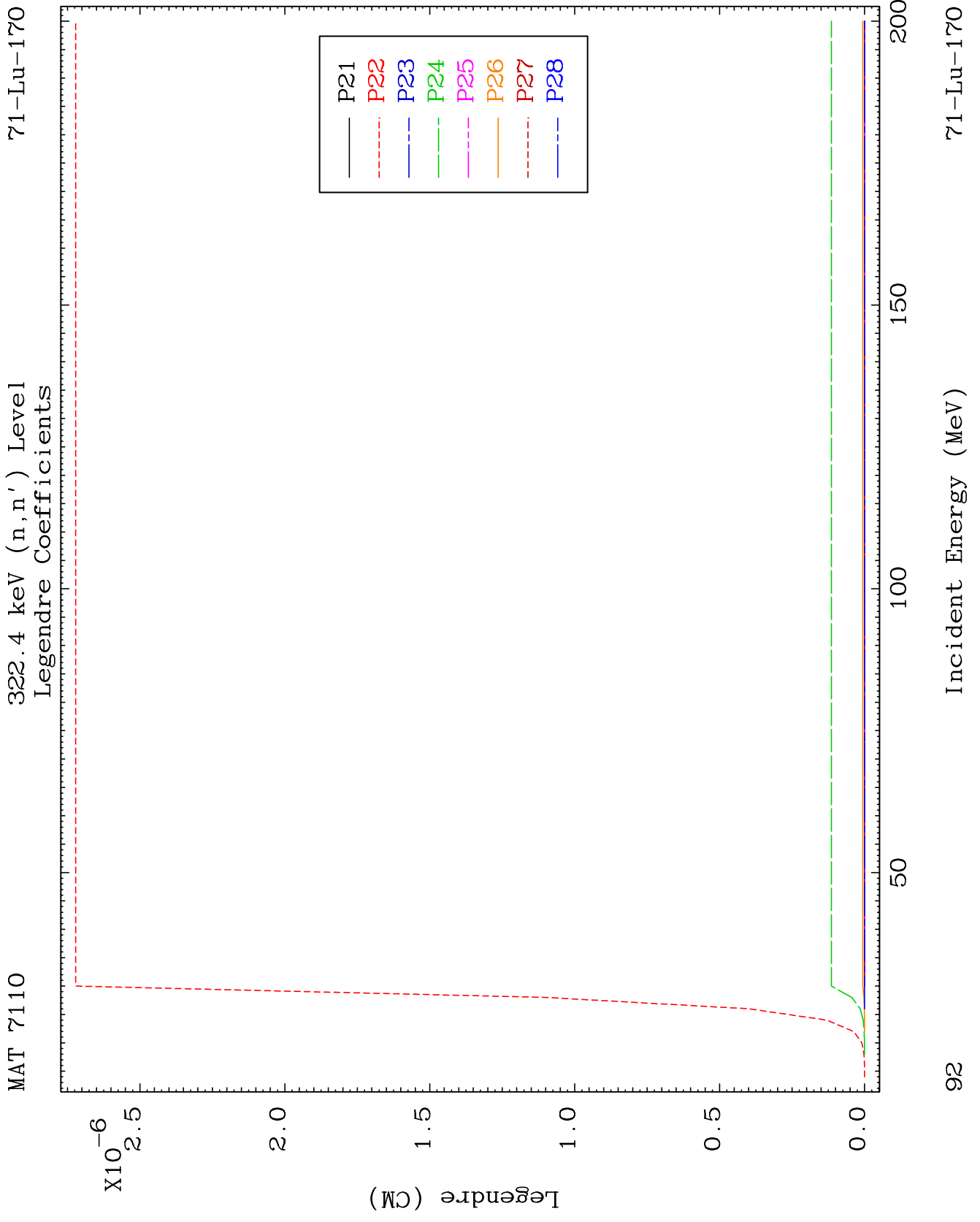








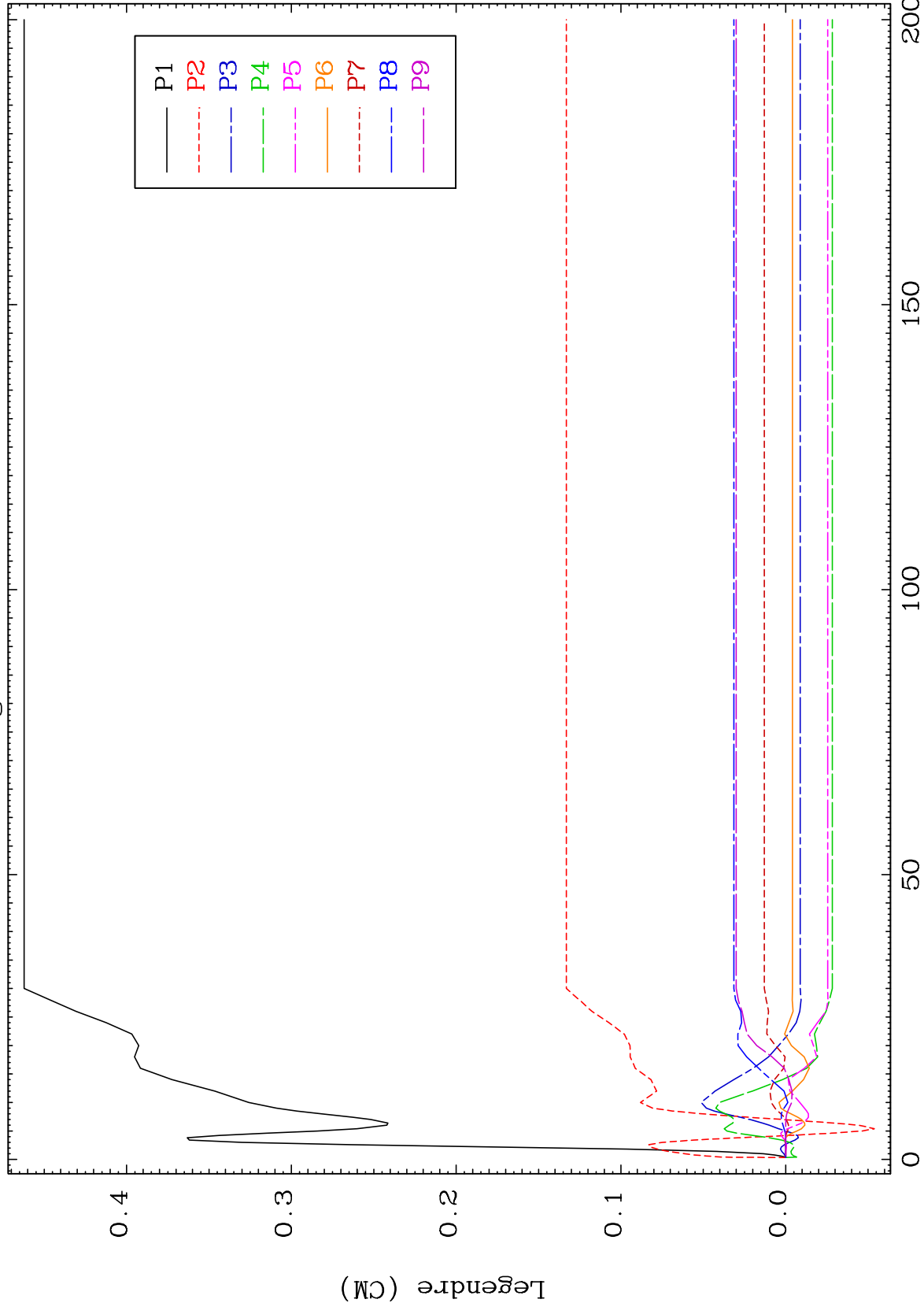




MAT 7110

328.9 keV (n,n') Level  
Legendre Coefficients

71-Lu-170



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Incident Energy (MeV)

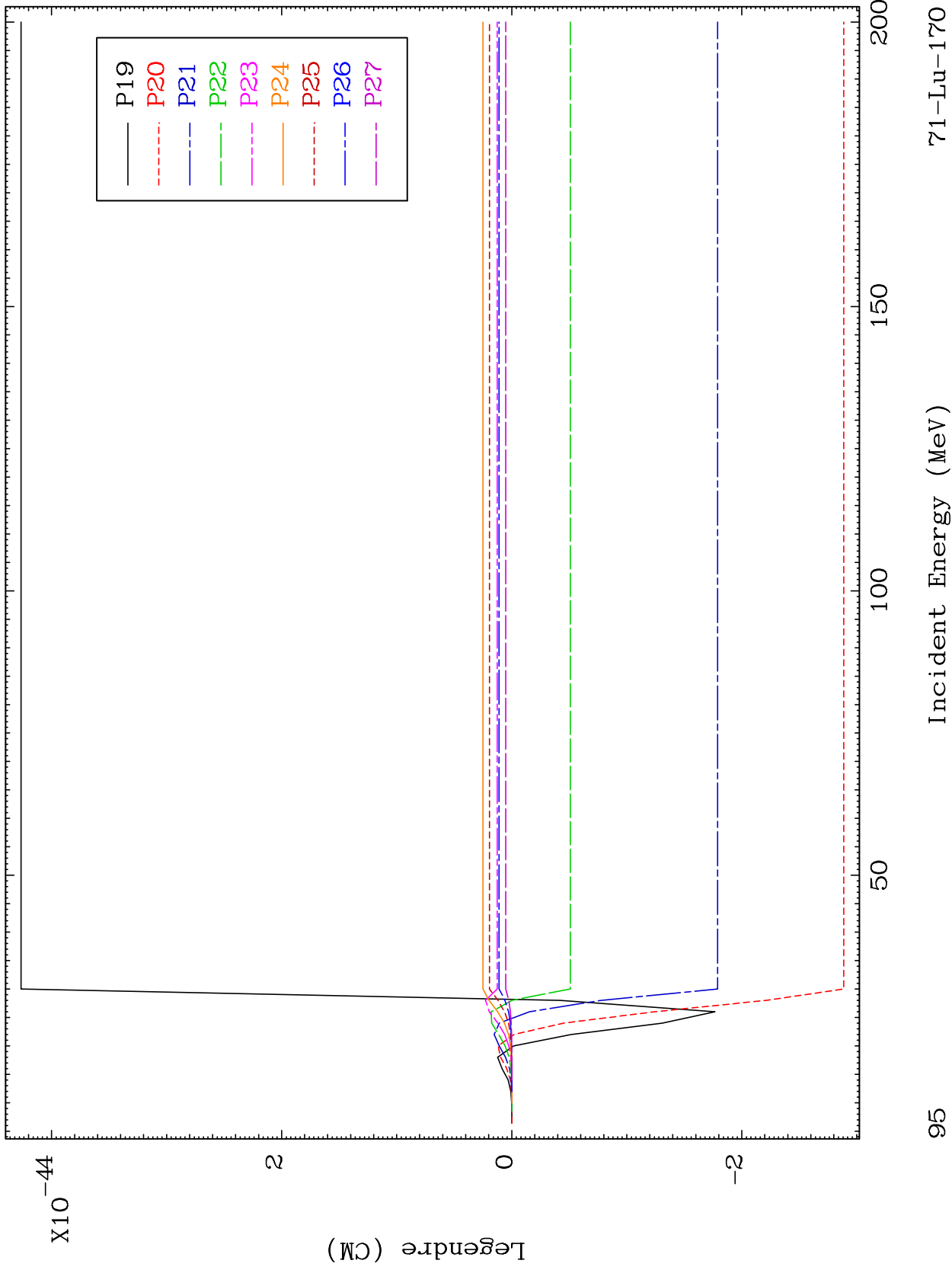
71-Lu-170



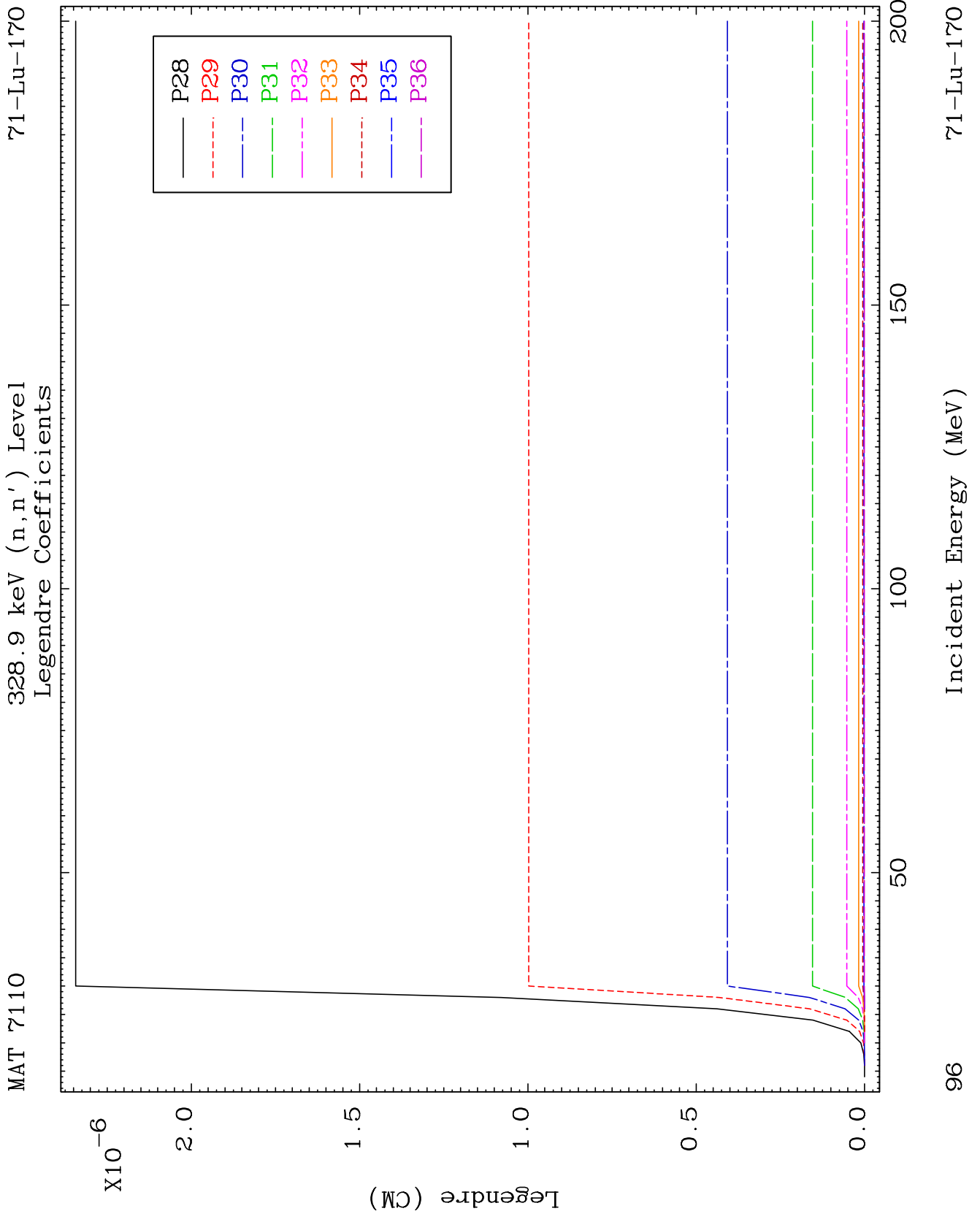
MAT 7110

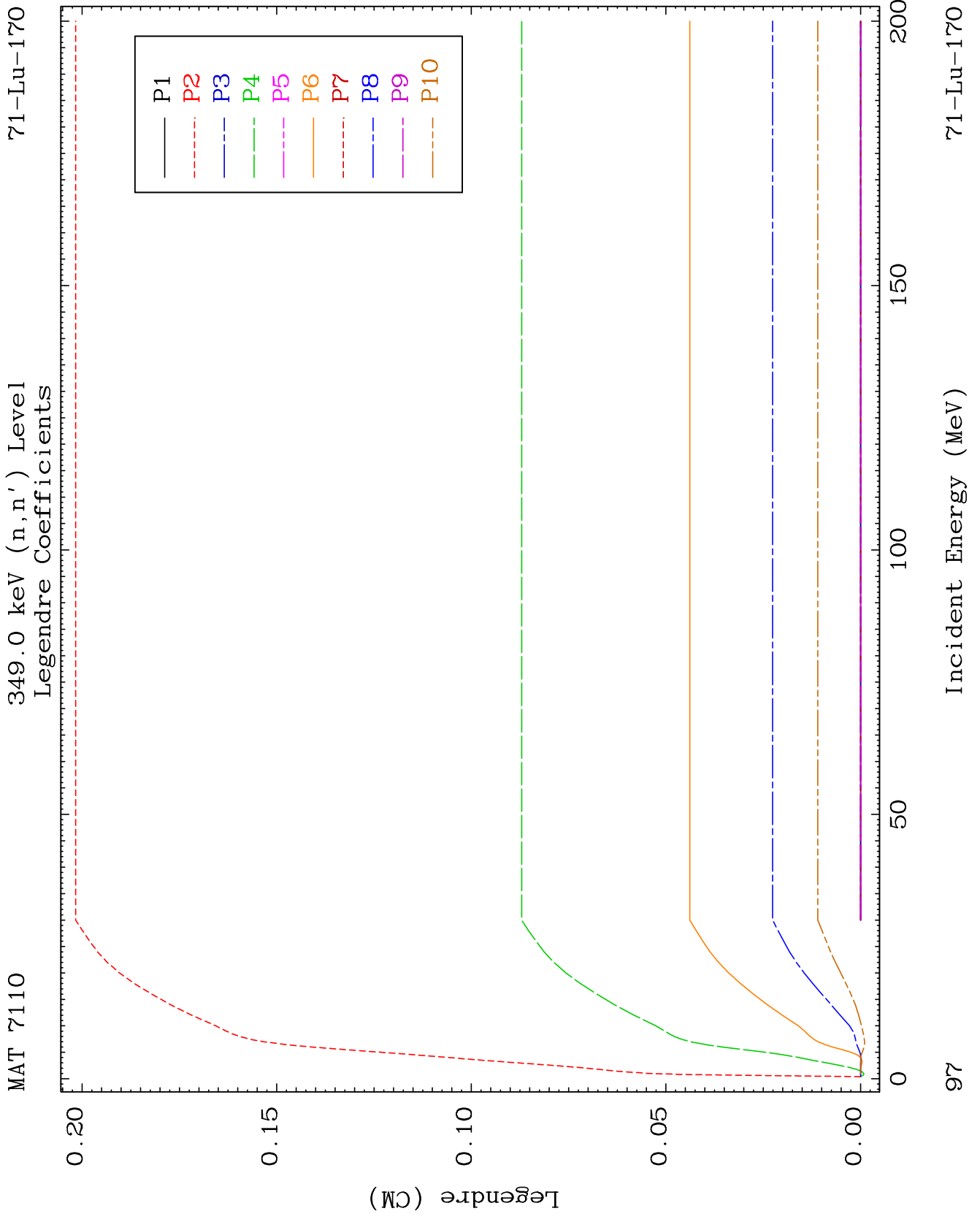
328.9 keV (n, n') Level  
Legendre Coefficients

71-Lu-170

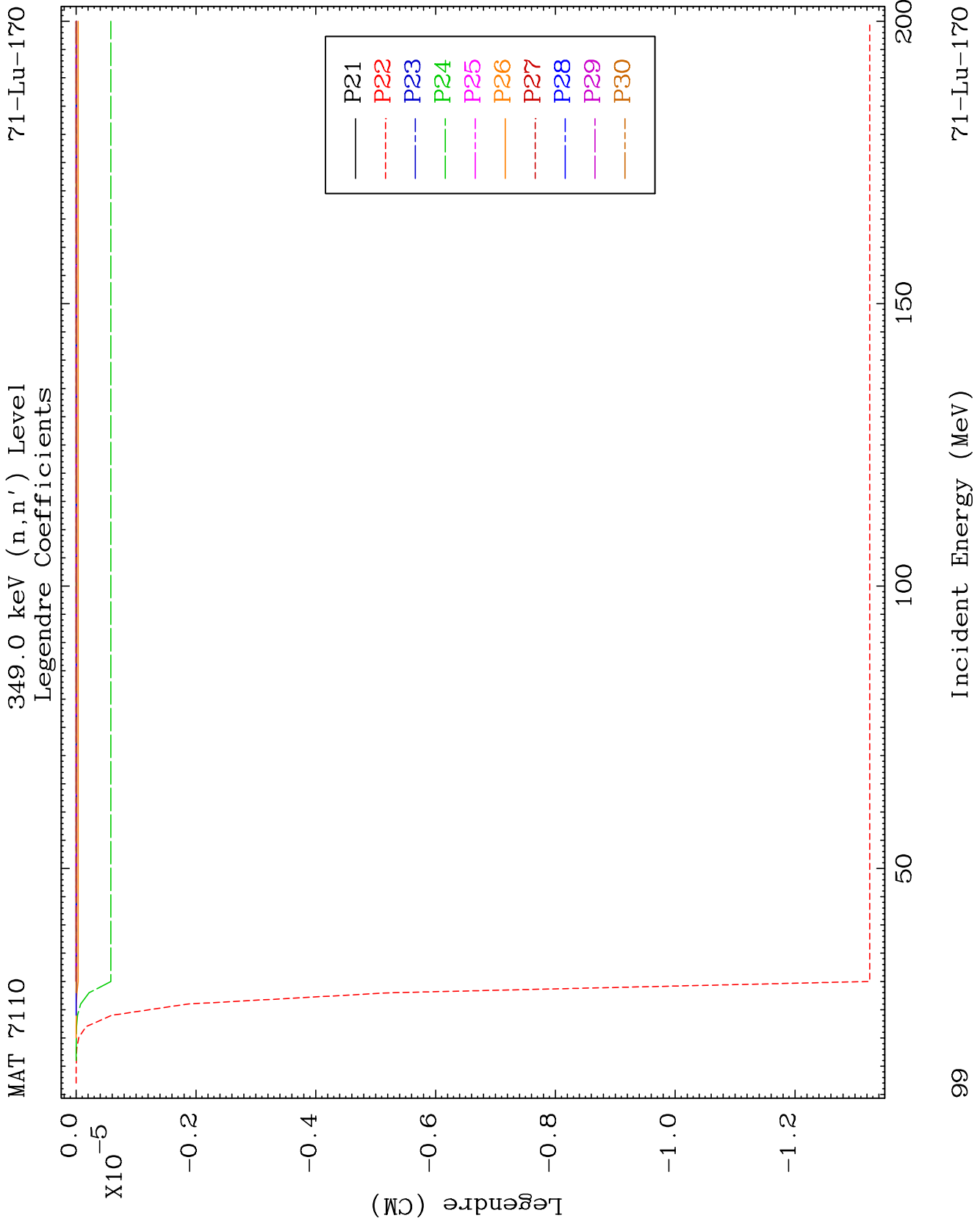


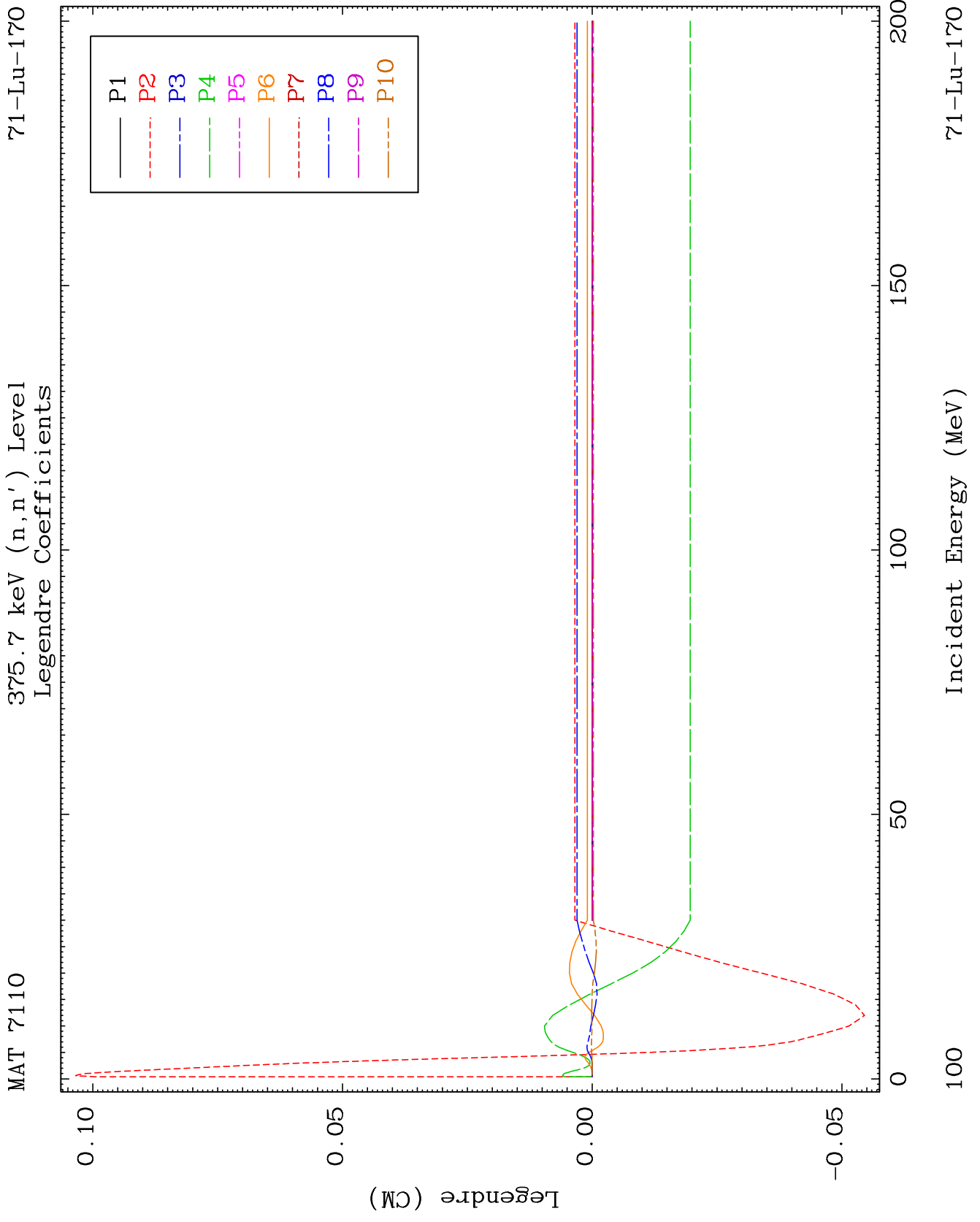






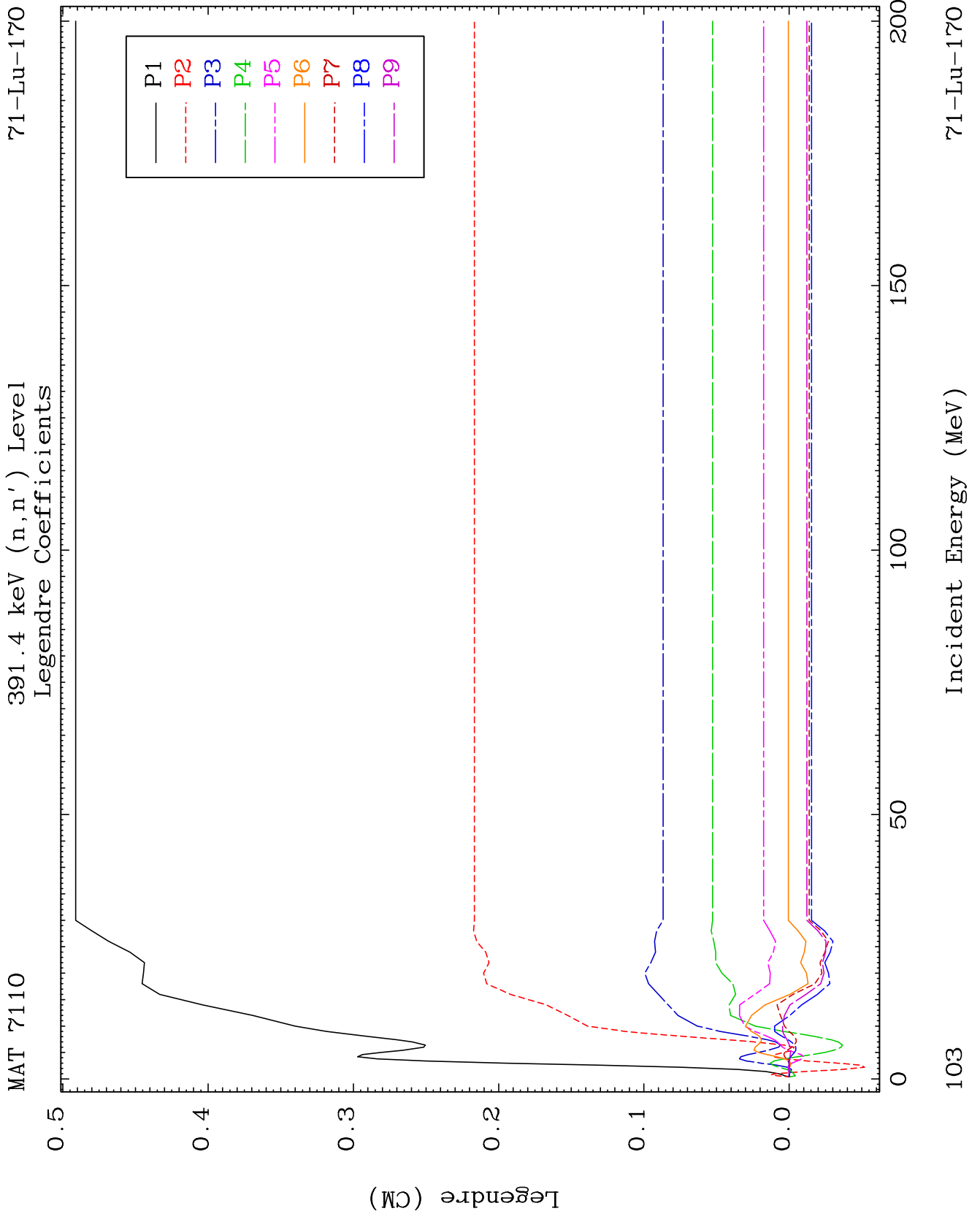










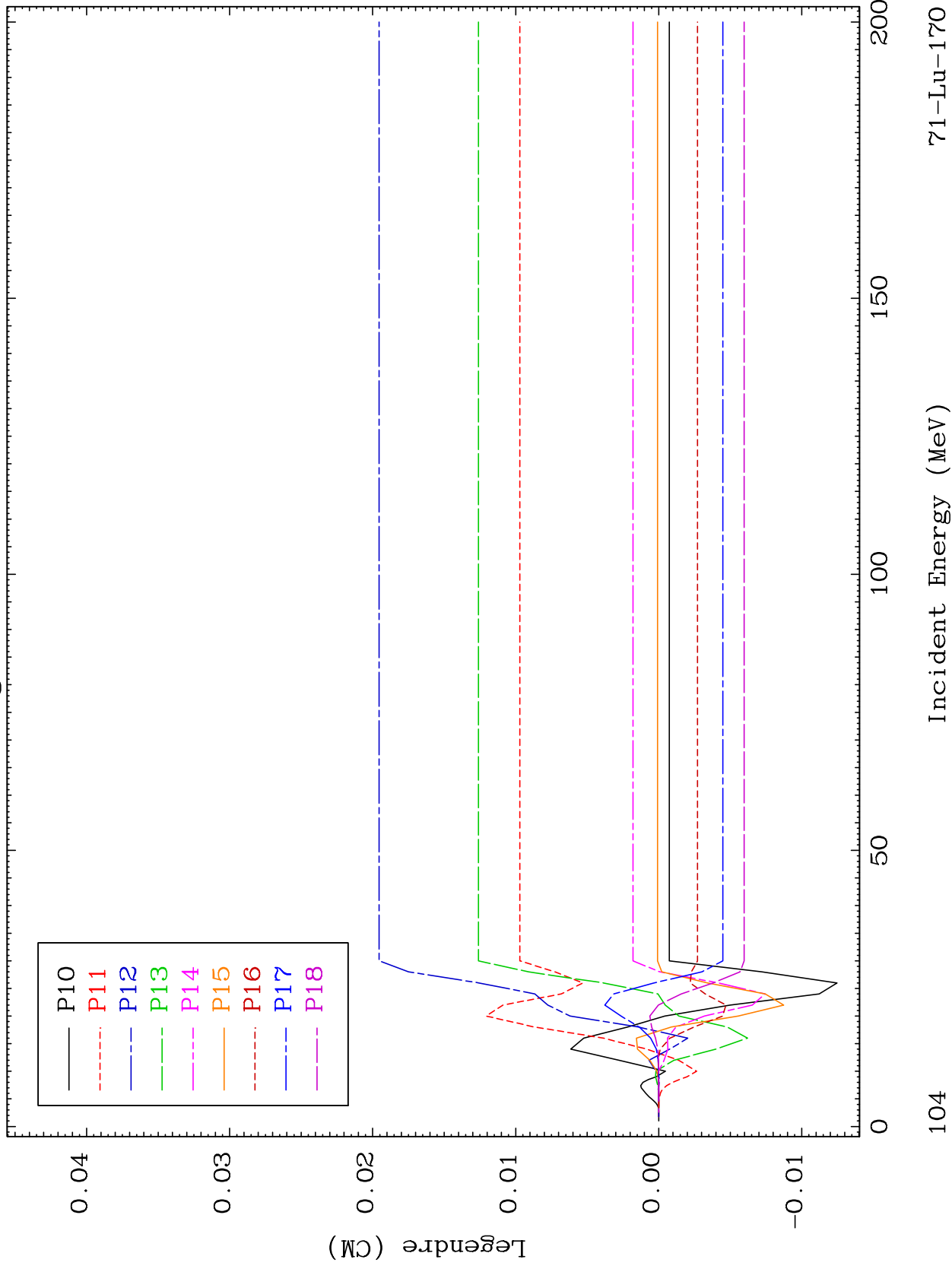




MAT 7110

391.4 keV (n,n') Level  
Legendre Coefficients

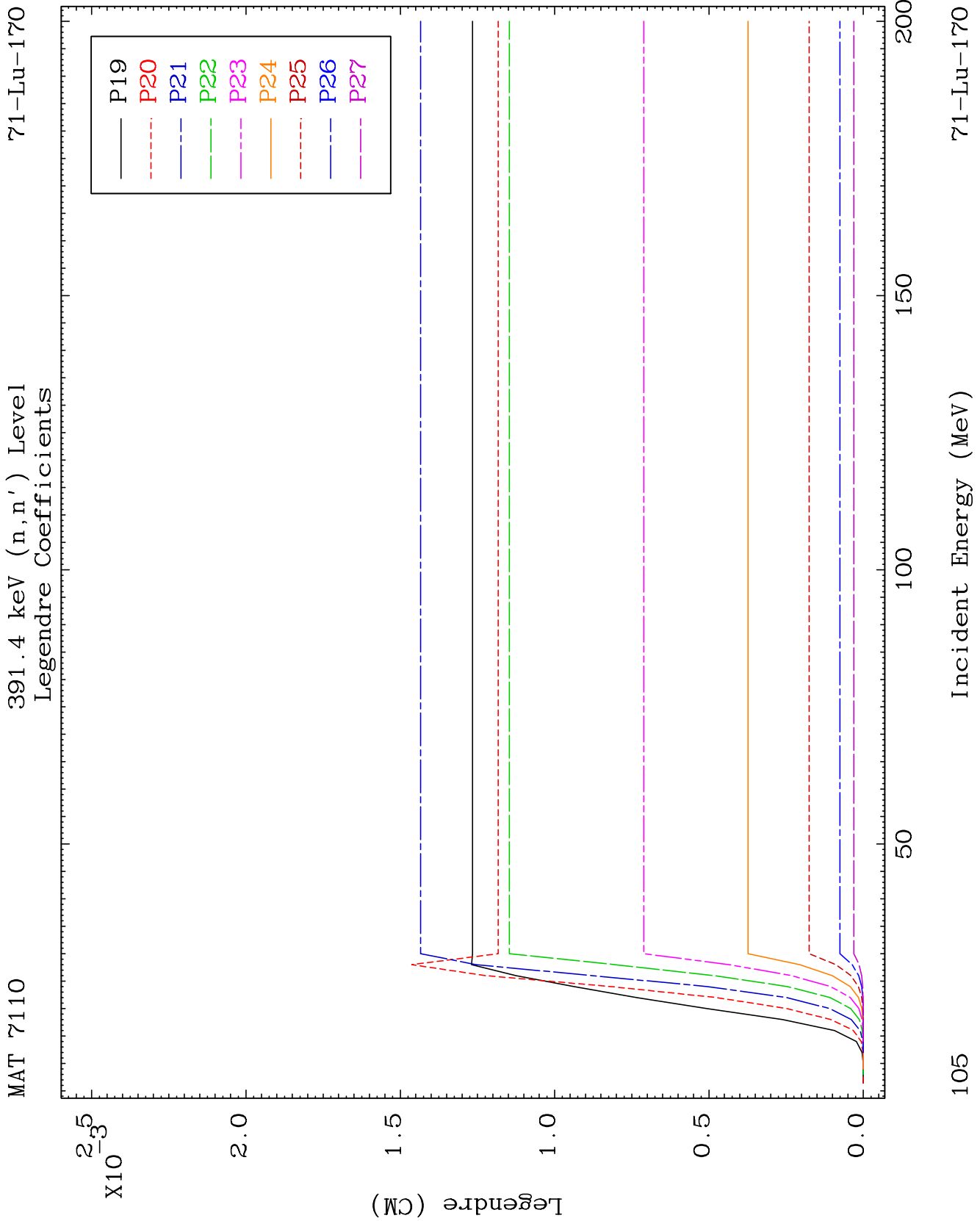
71-Lu-170

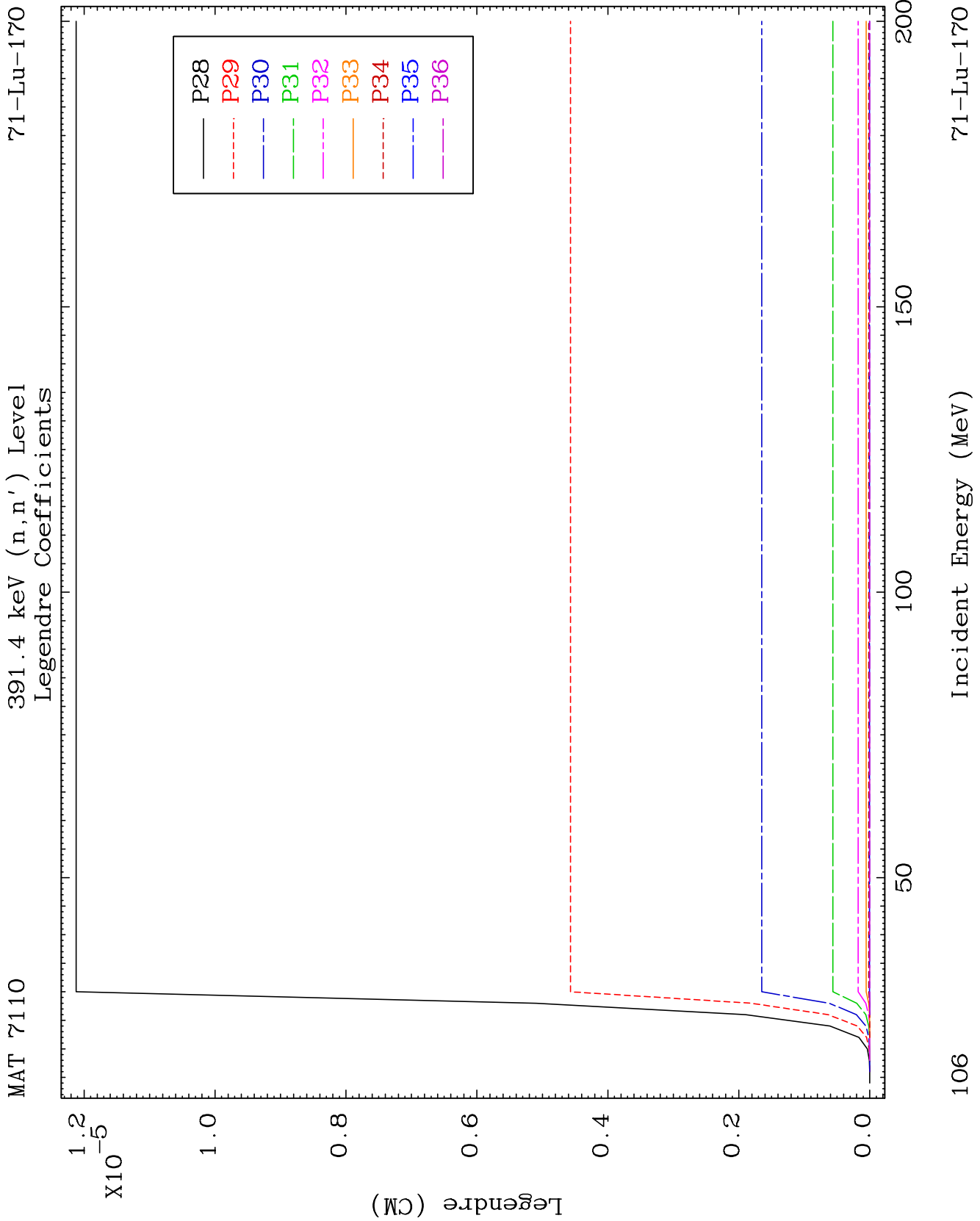


104

Incident Energy (MeV)

71-Lu-170



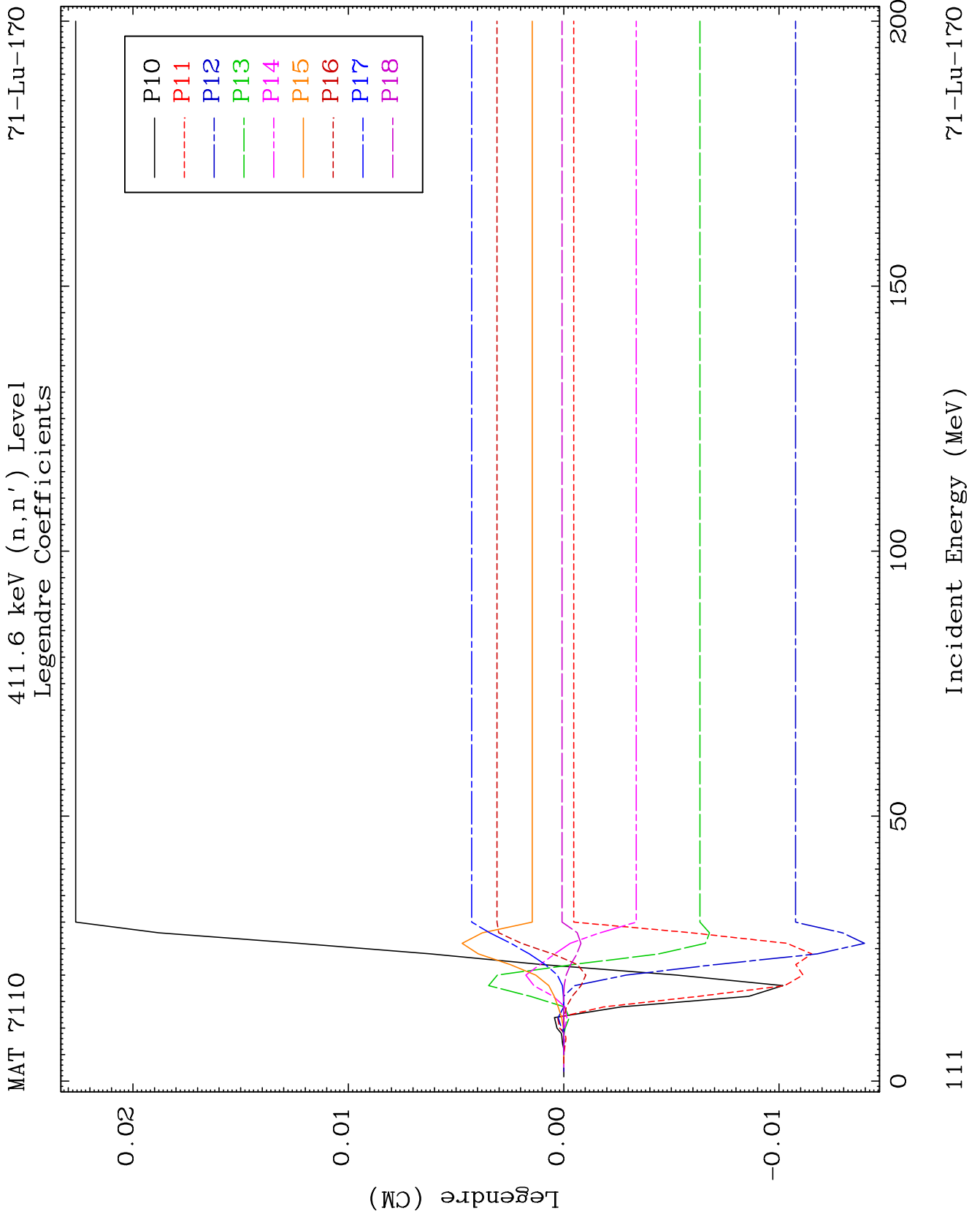




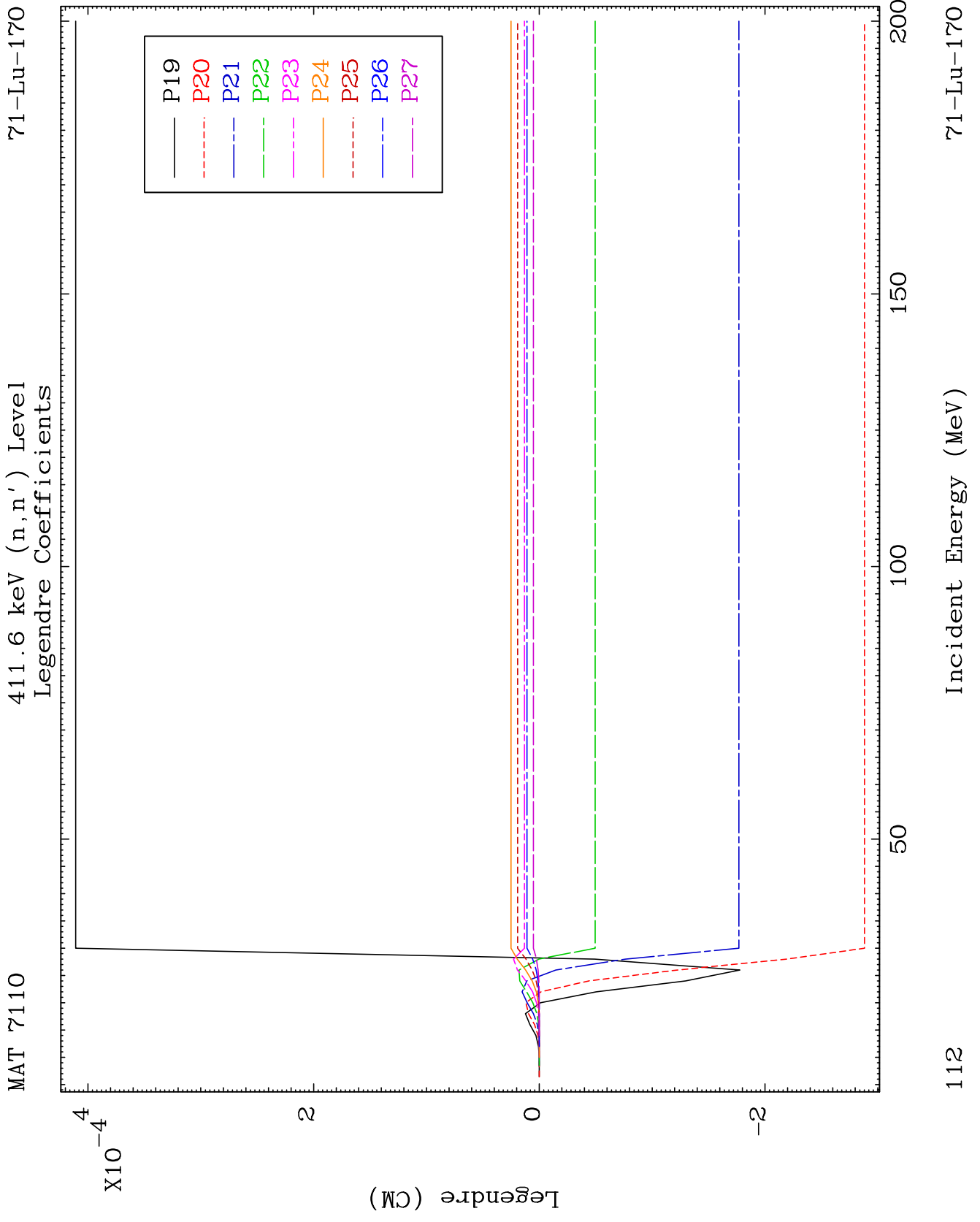




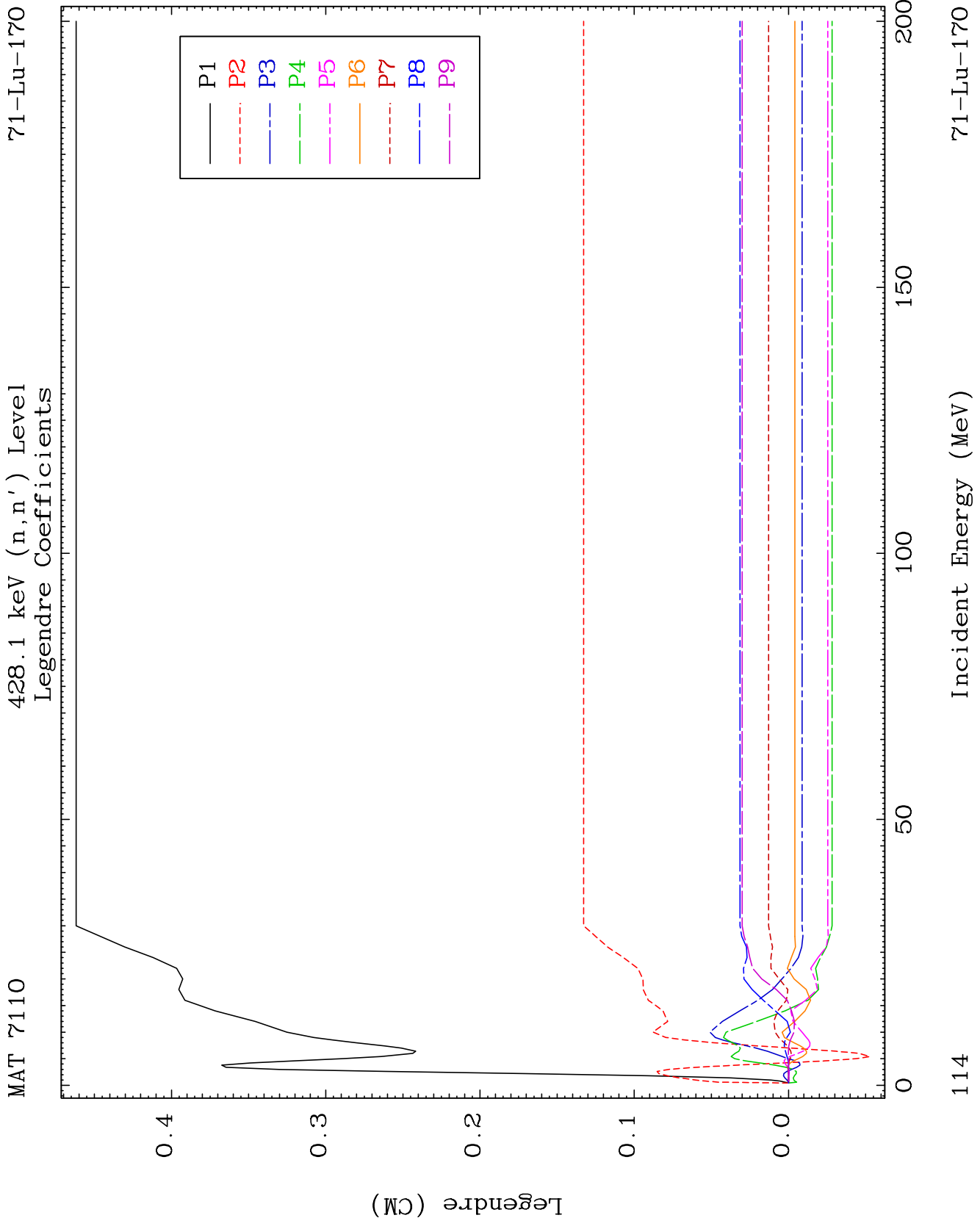


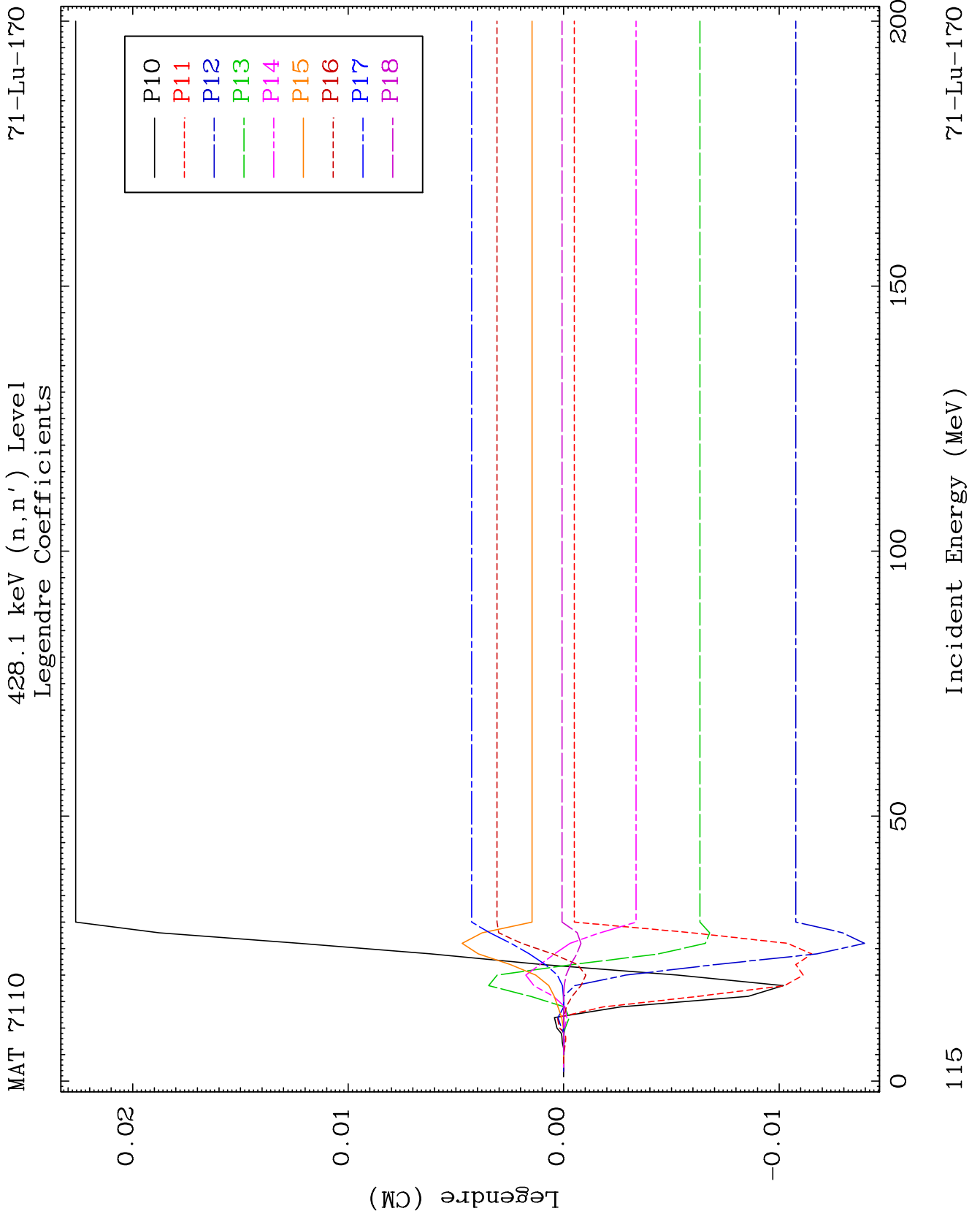


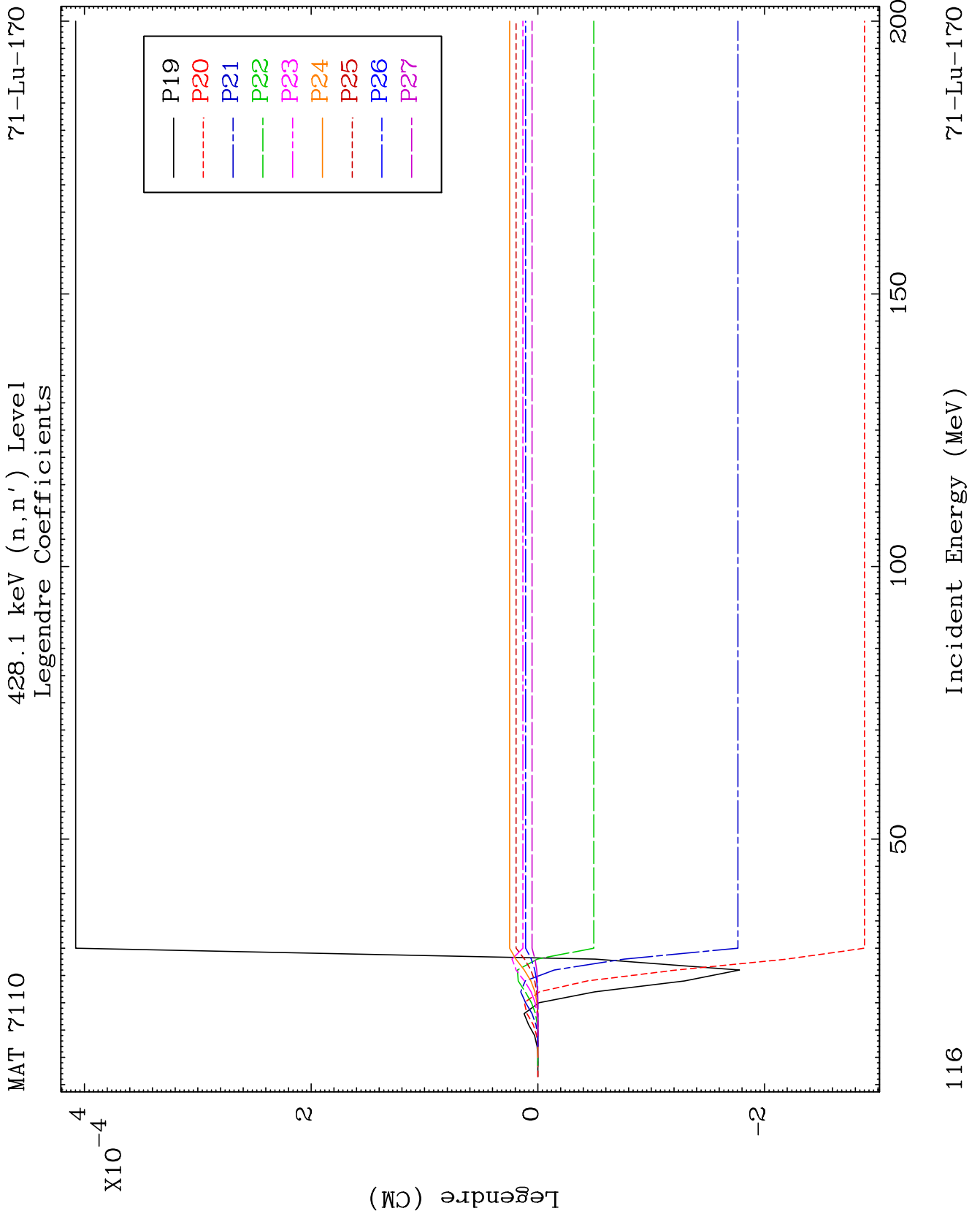


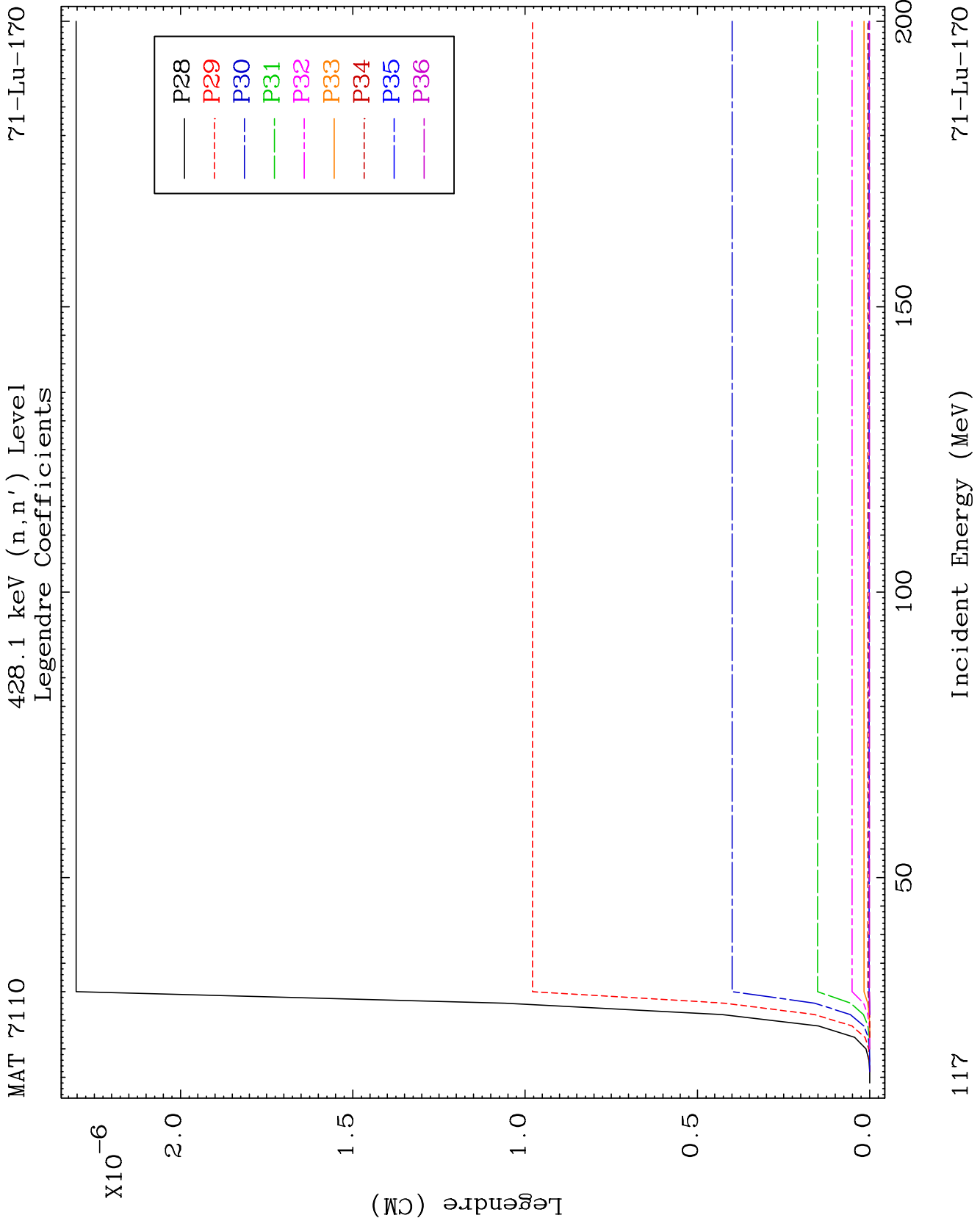


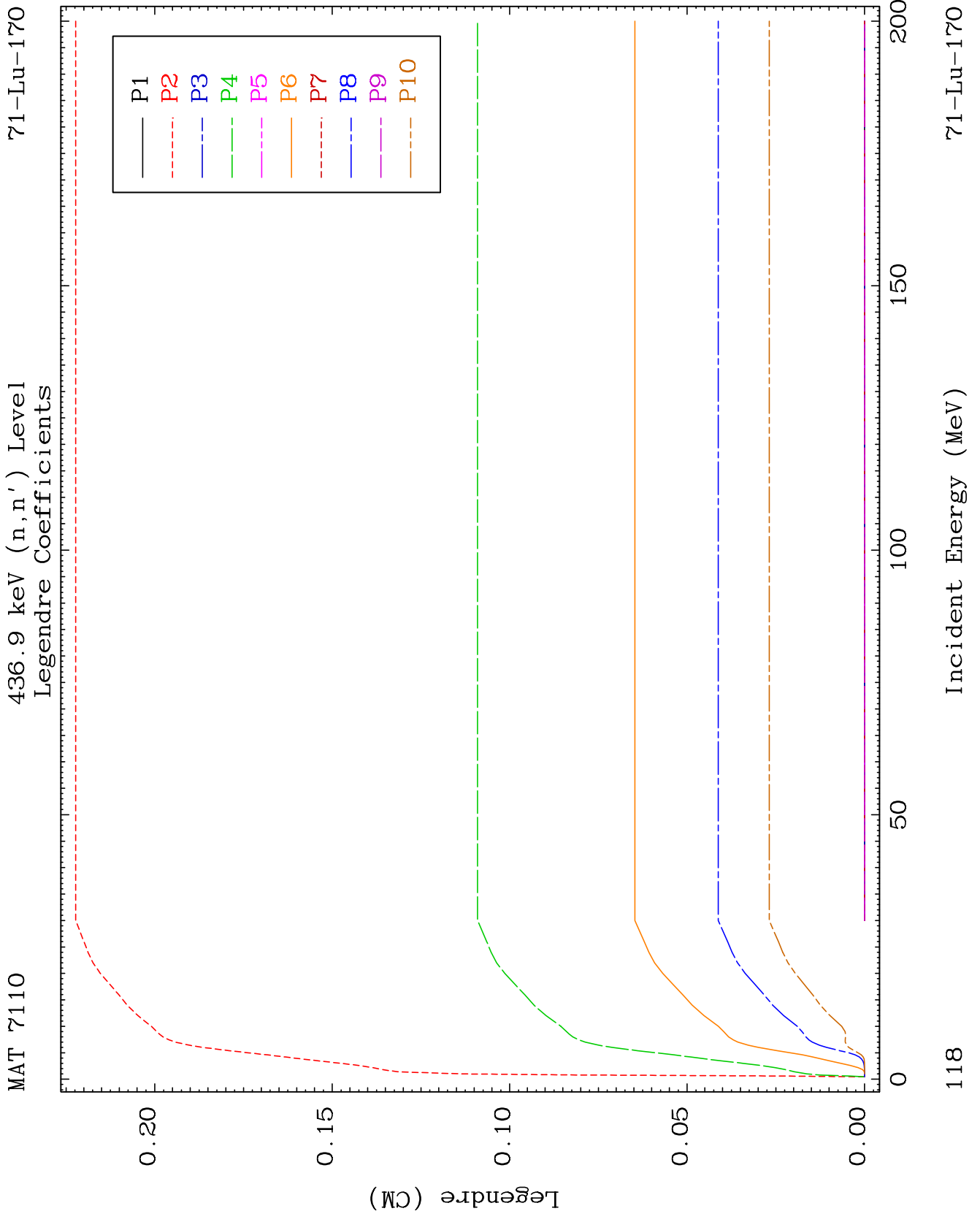


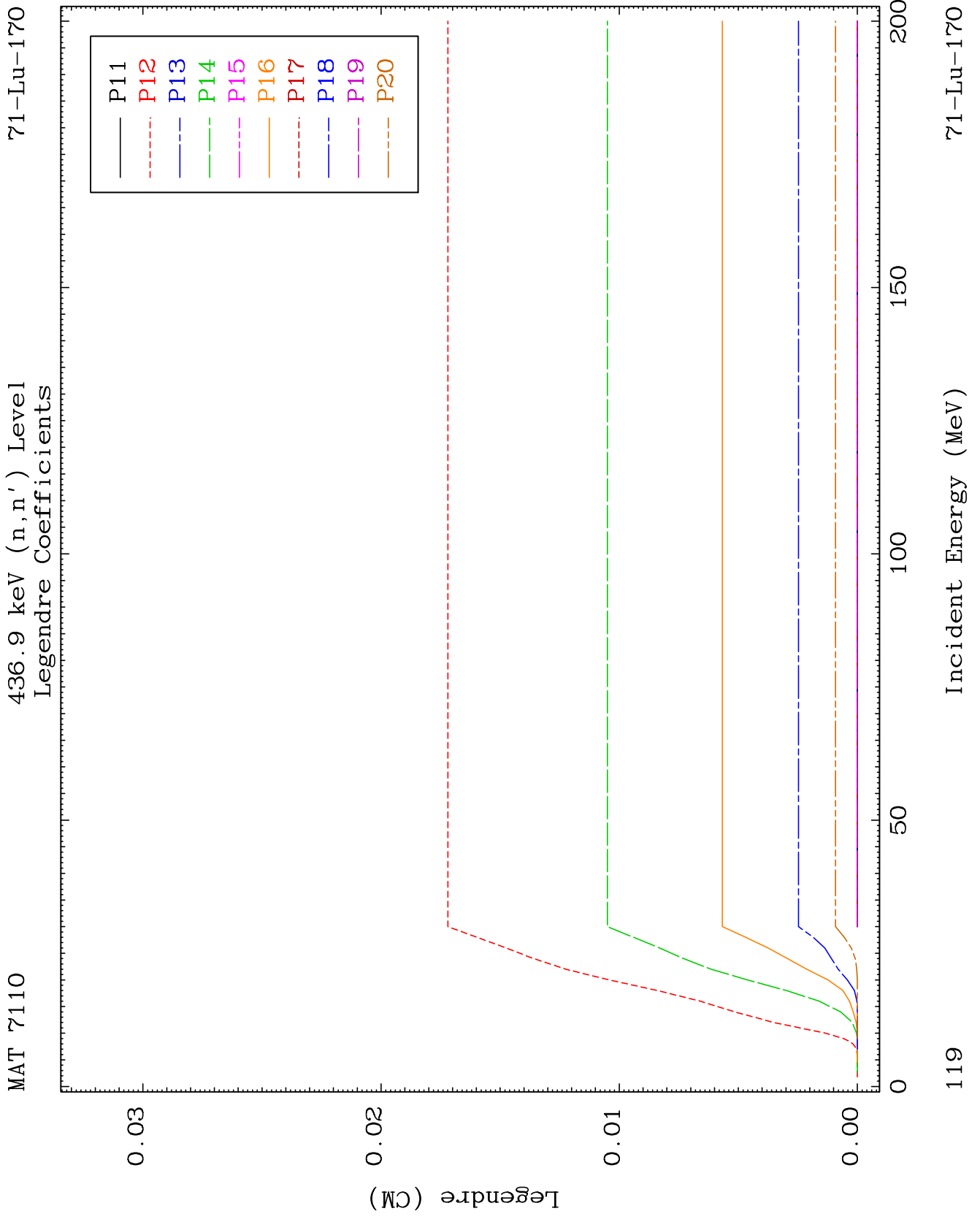




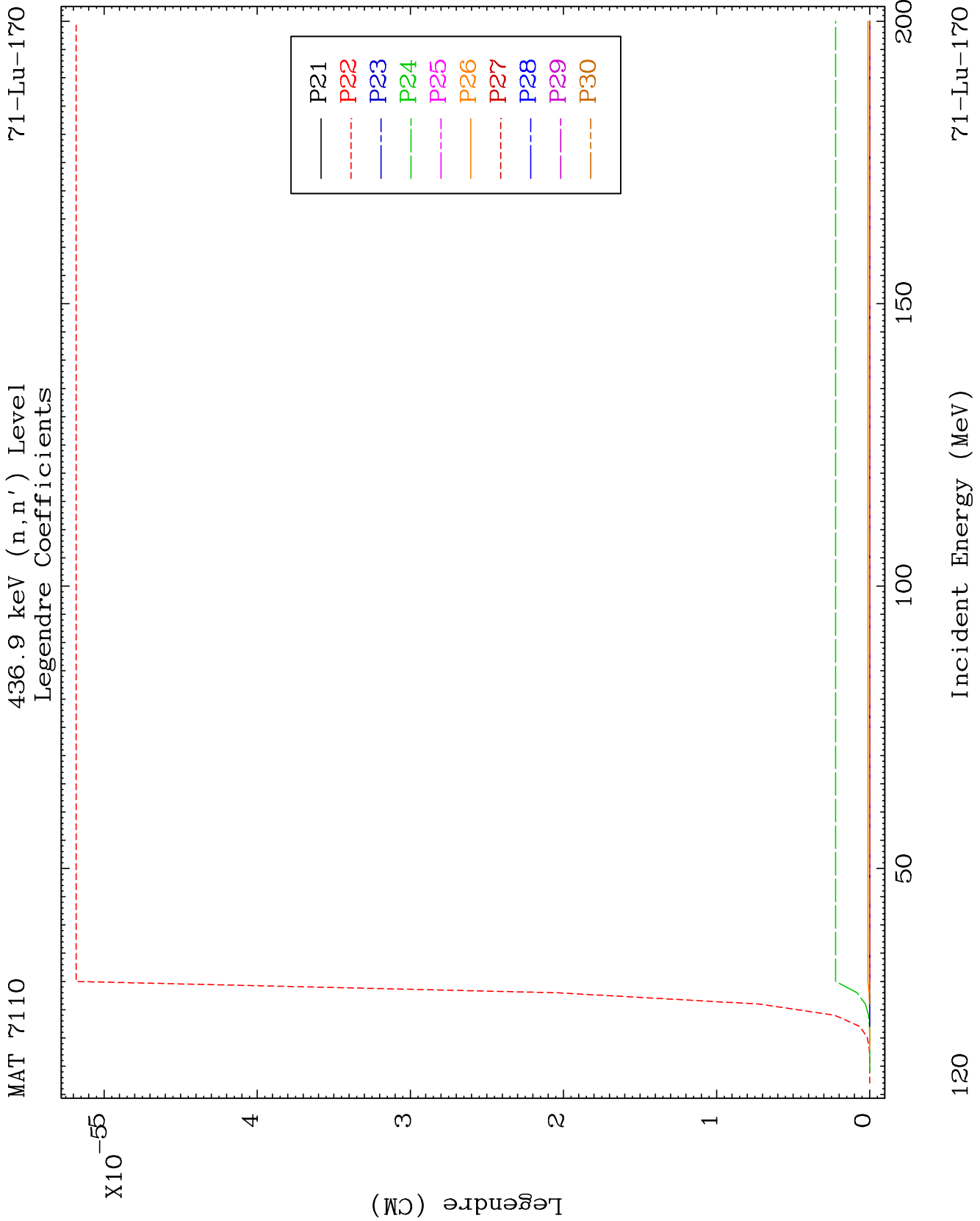


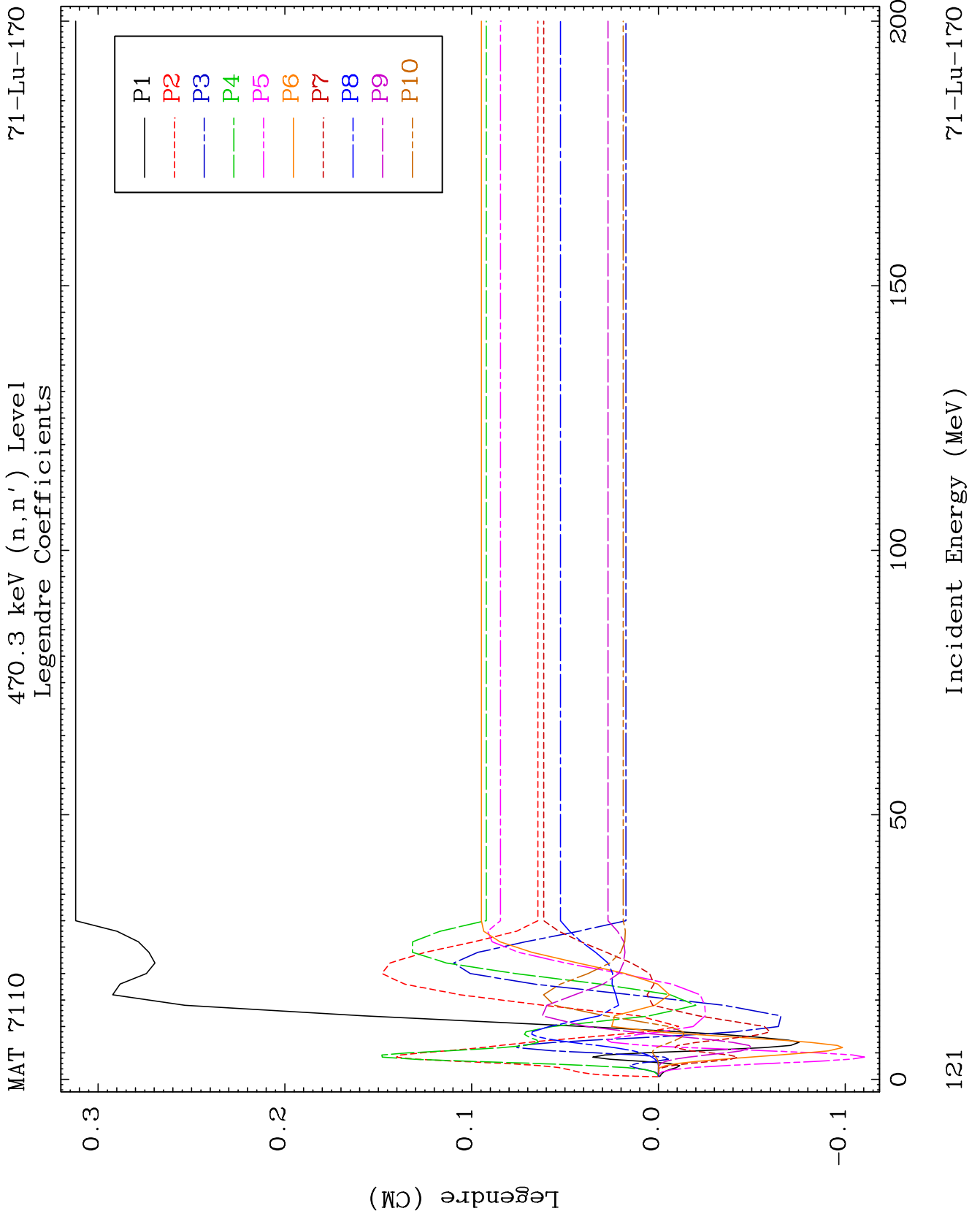


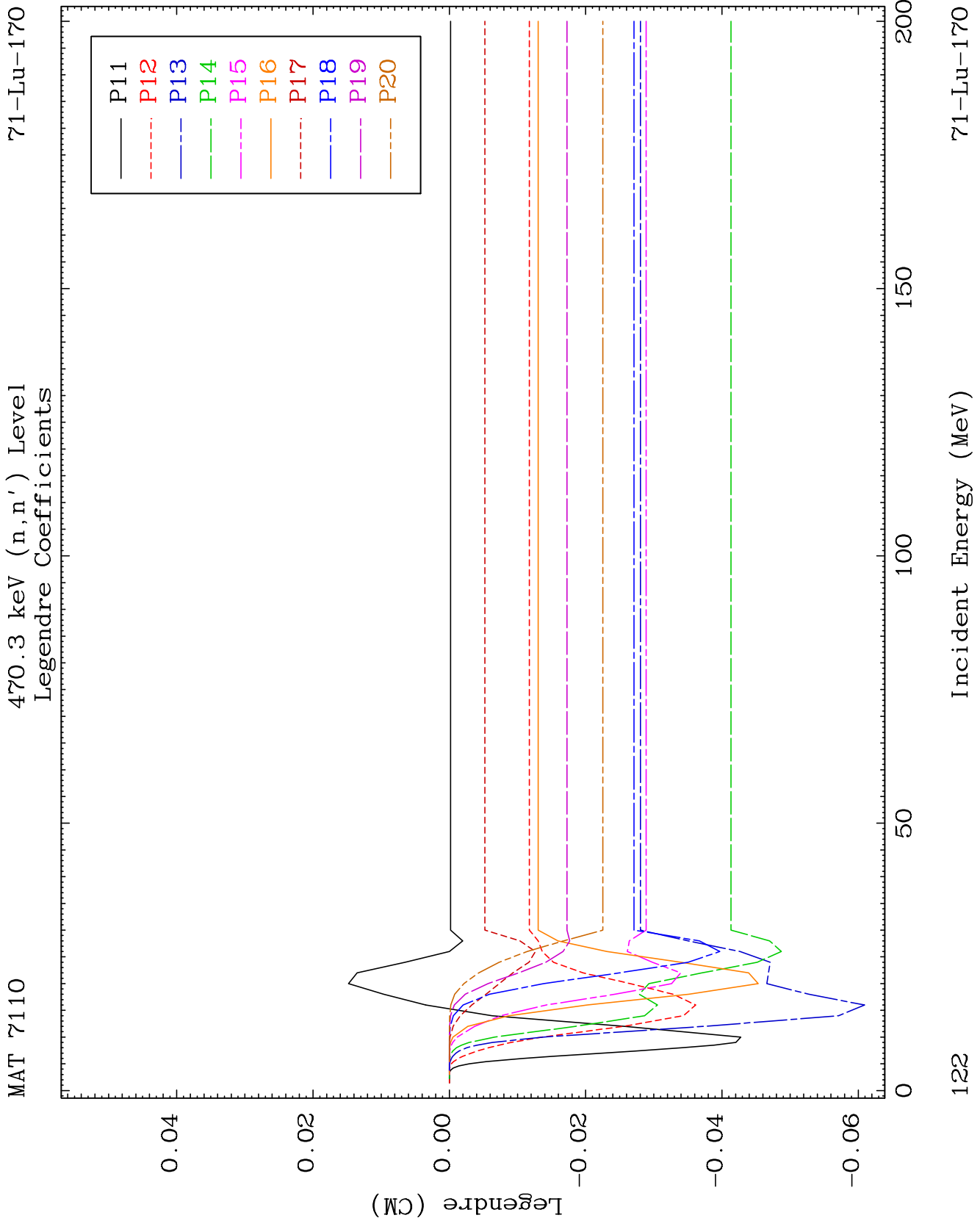


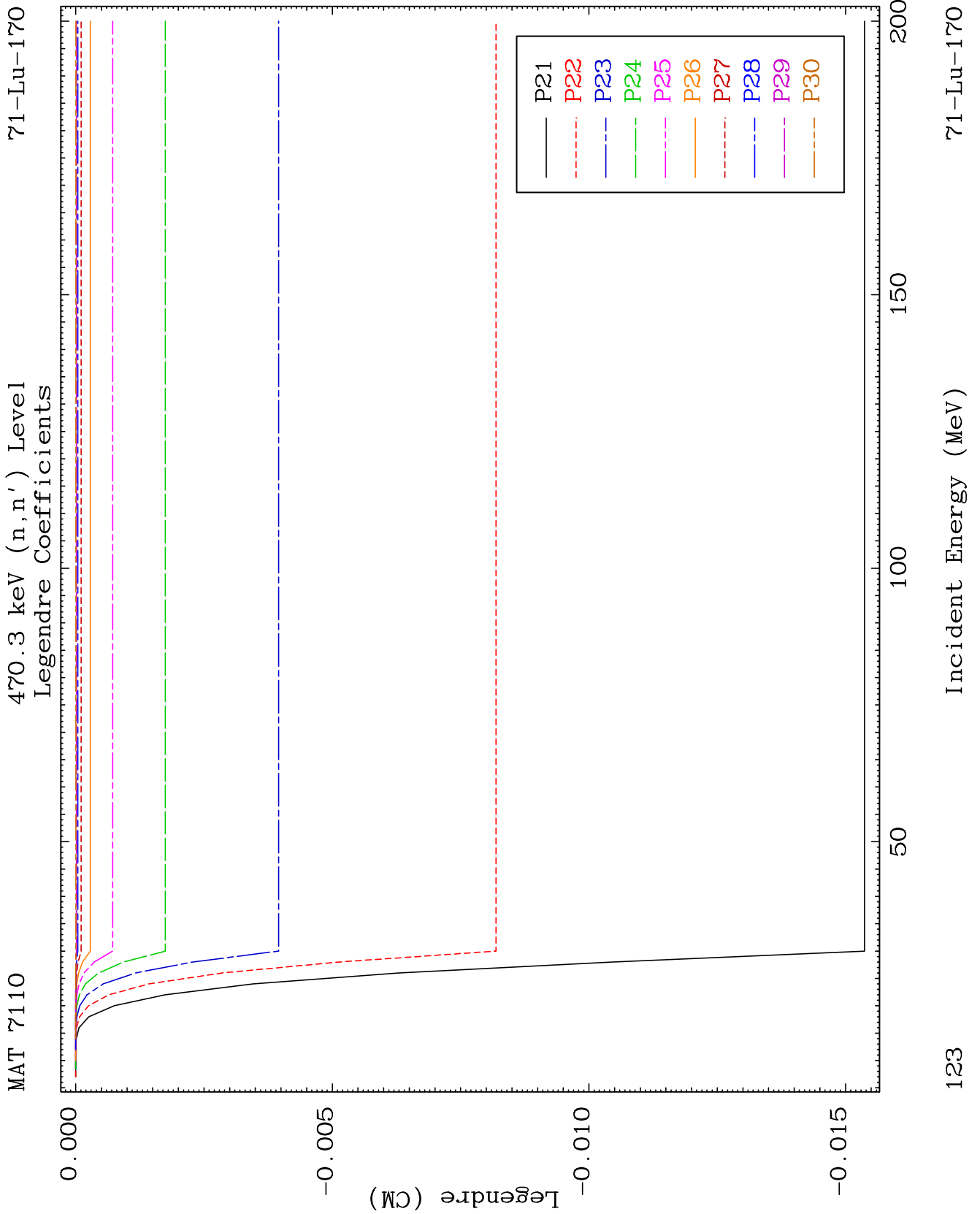










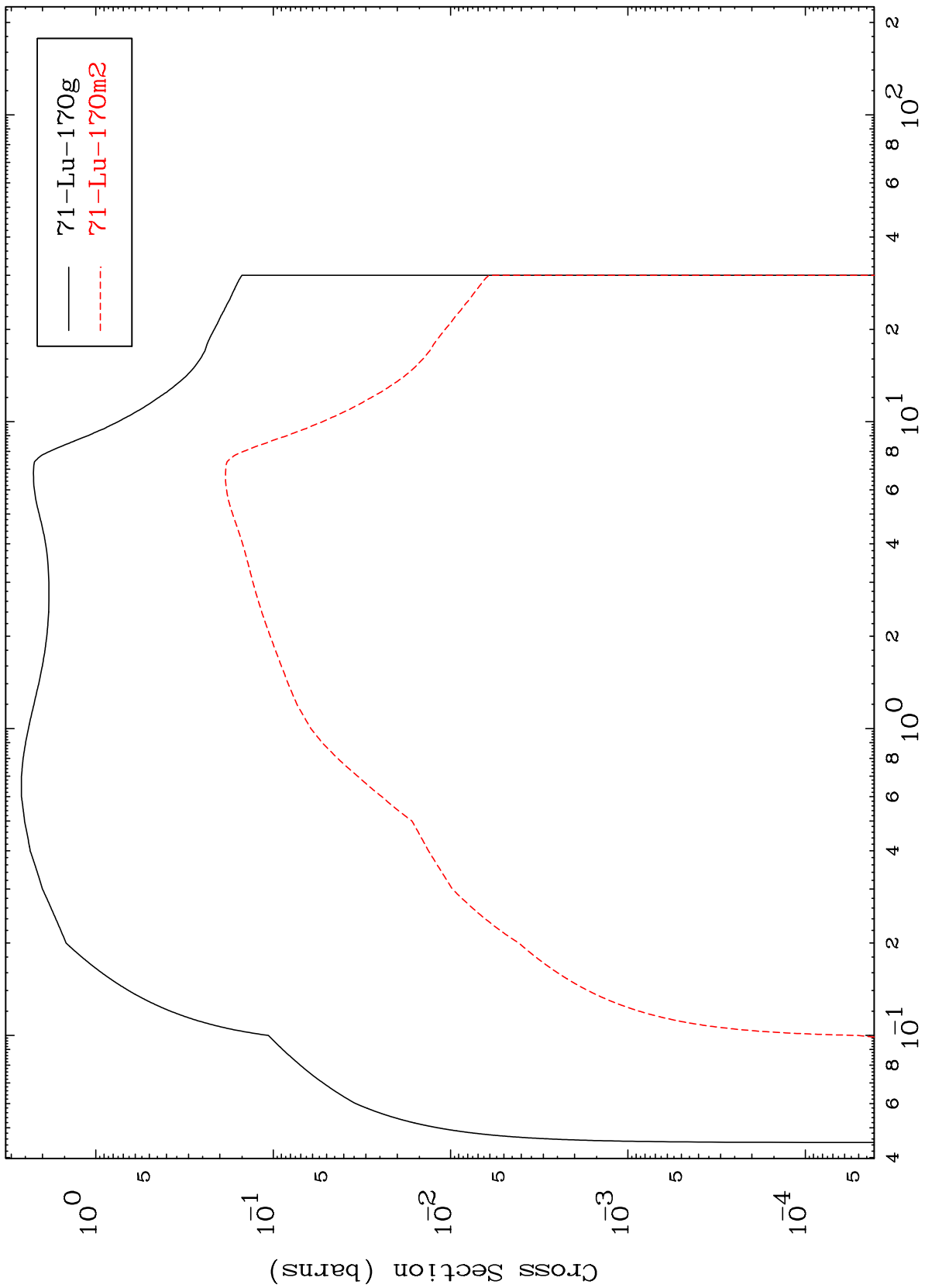




MAT 7110

71-Lu-170

Inelastic  
Radionuclide Production Cross Section



125

Incident Energy (MeV)

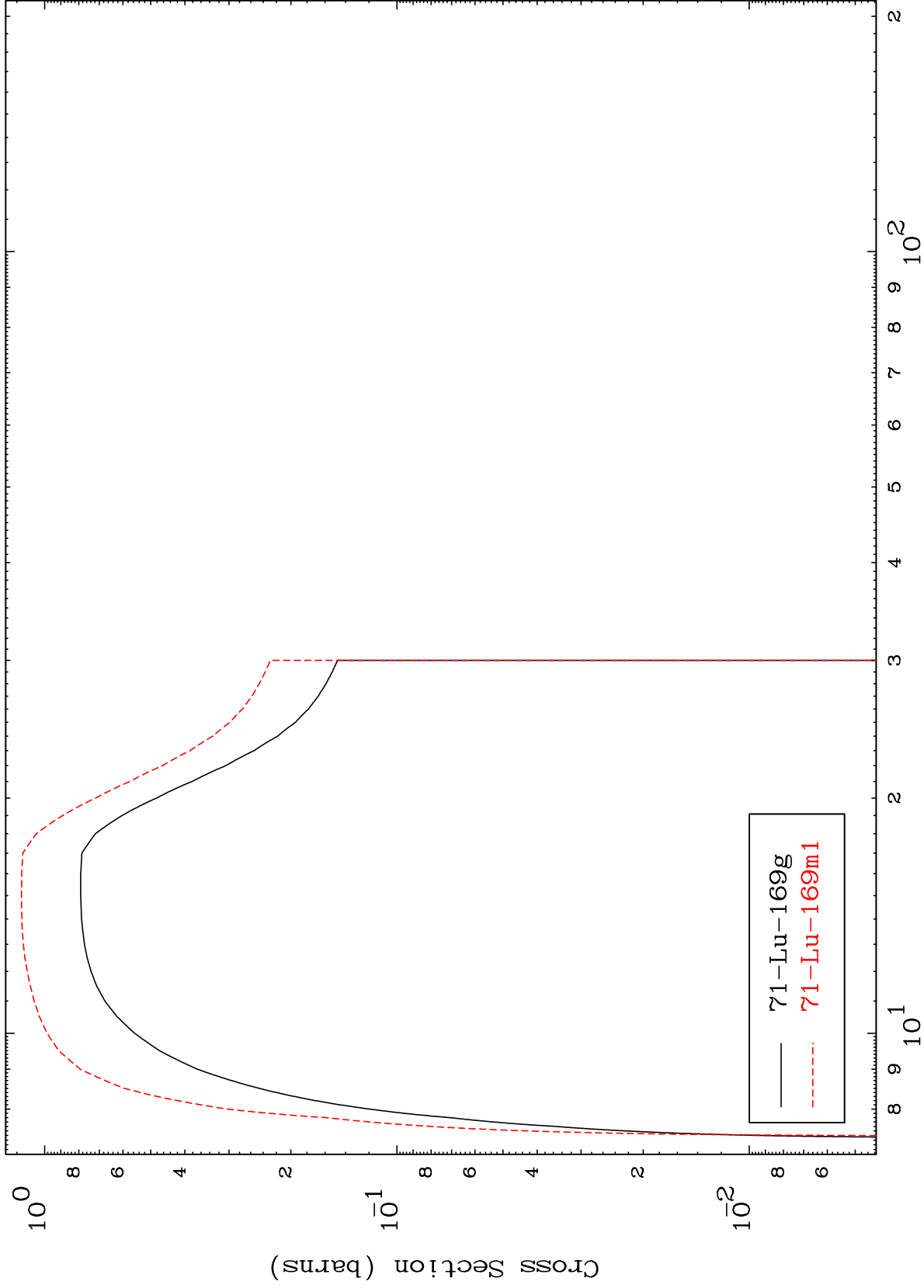
71-Lu-170

MAT 7110

(n,2n)

71-Lu-170

Radionuclide Production Cross Section



126

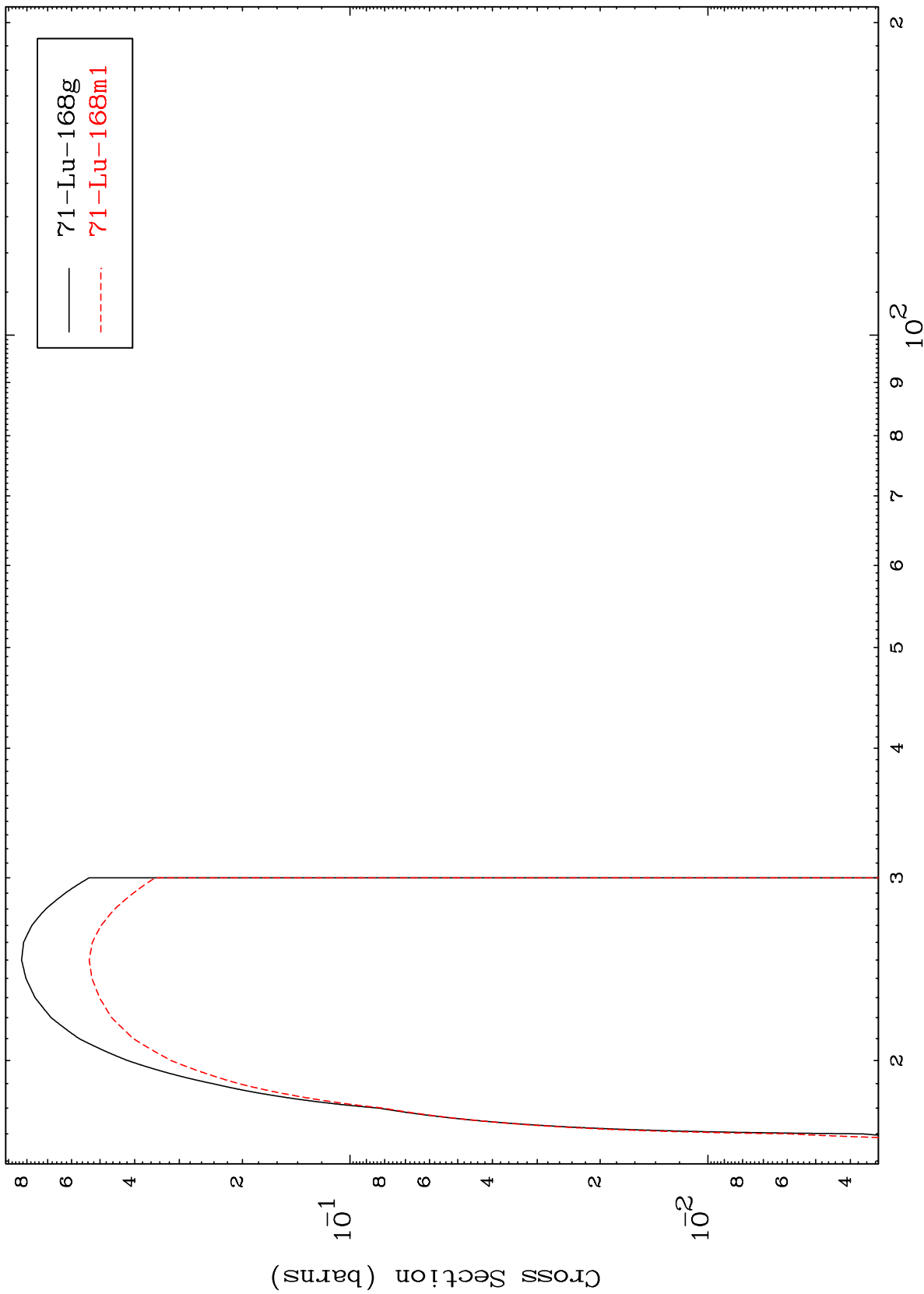
Incident Energy (MeV)

71-Lu-170

MAT 7110

71-Lu-170

(n,3n)  
Radionuclide Production Cross Section



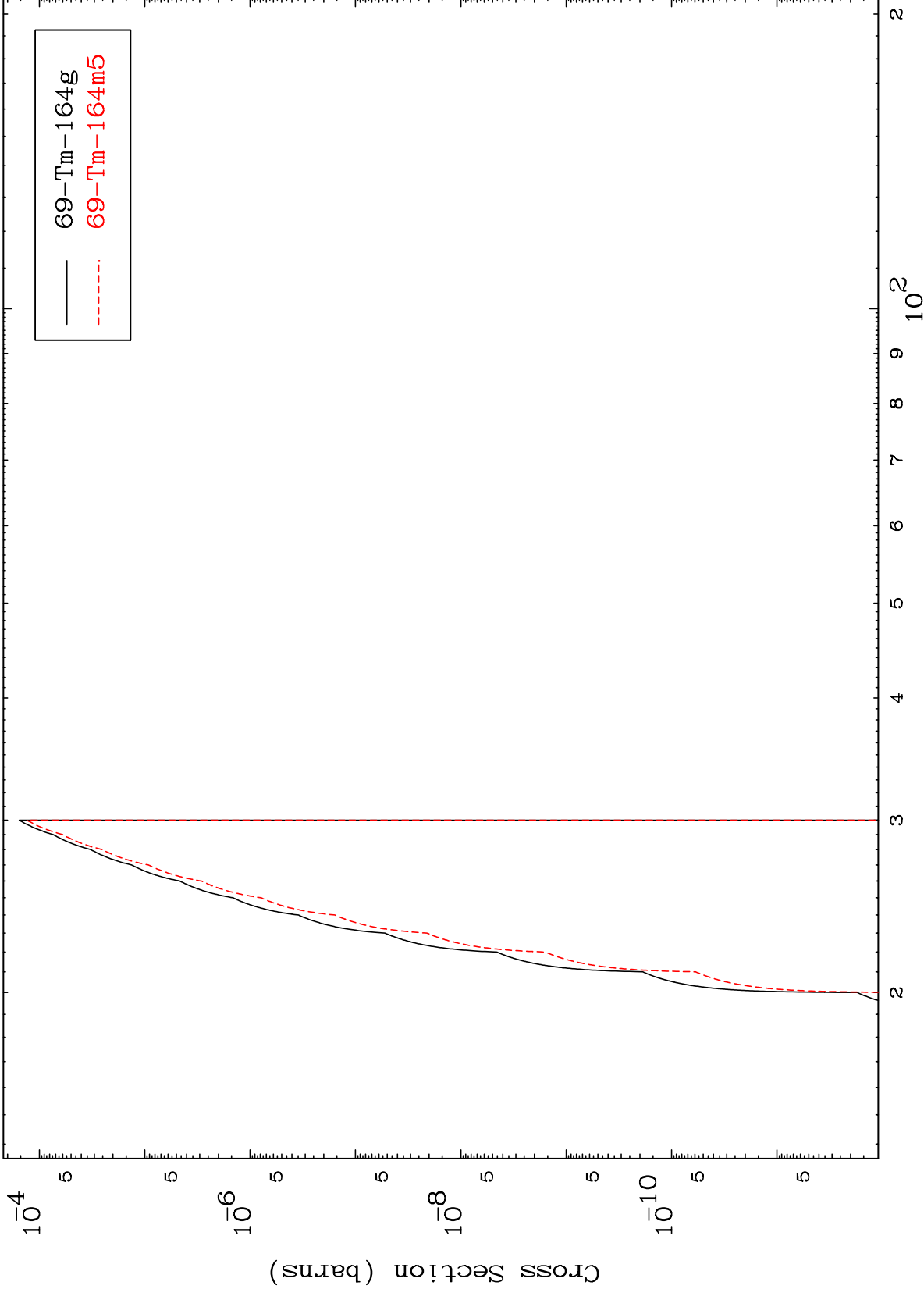
127

Incident Energy (MeV)

71-Lu-170



Radionuclide Production Cross Section

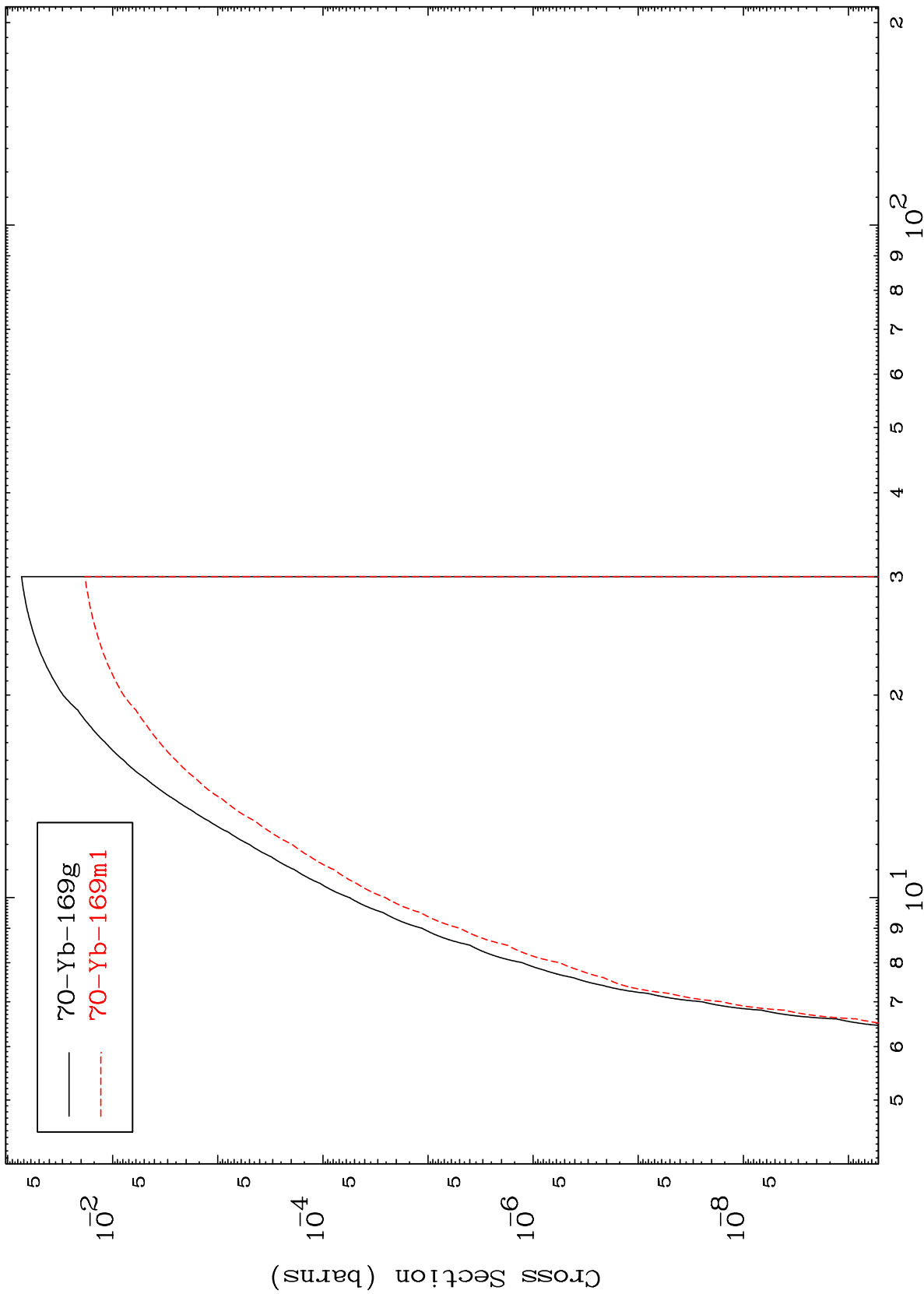


MAT 7110

(n,n') p

71-Lu-170

Radionuclide Production Cross Section



70-Yb-169g  
70-Yb-169m1

129

Incident Energy (MeV)

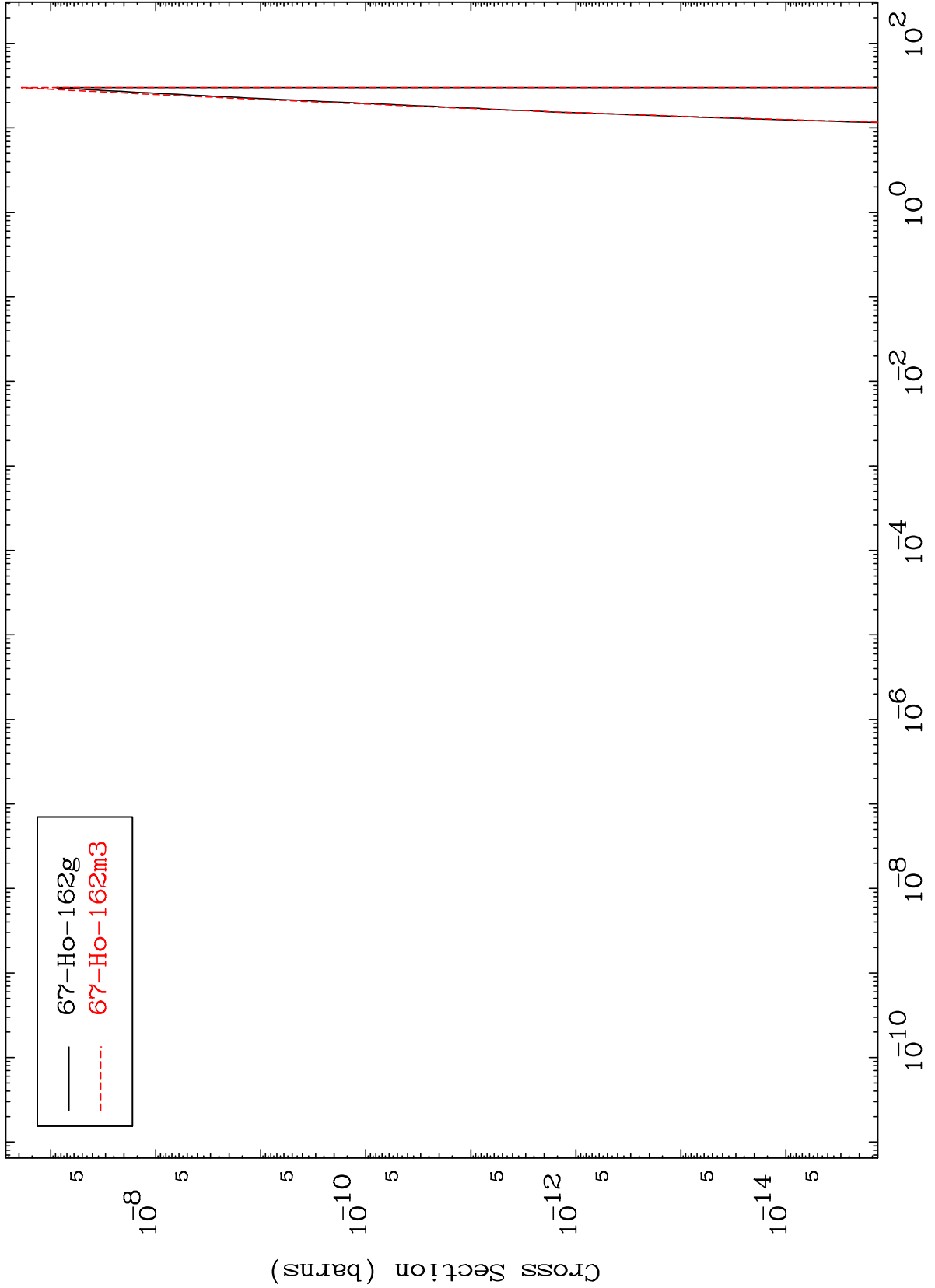
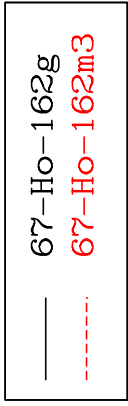
71-Lu-170

MAT 7110

(n,n') 2α

71-Lu-170

Radionuclide Production Cross Section



130

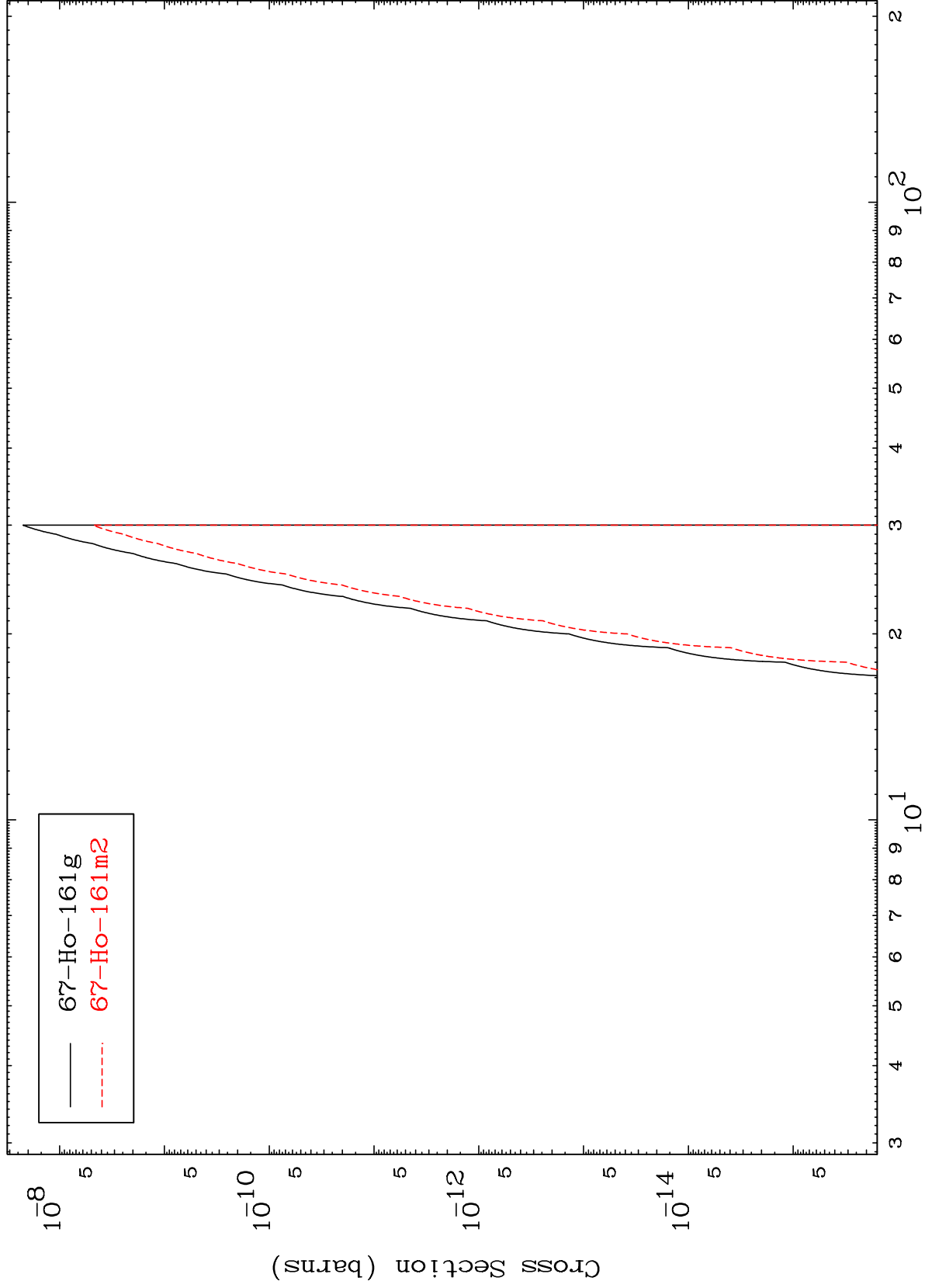
71-Lu-170

MAT 7110

$(n,2n) 2\alpha$

$^{71}\text{Lu-170}$

Radionuclide Production Cross Section

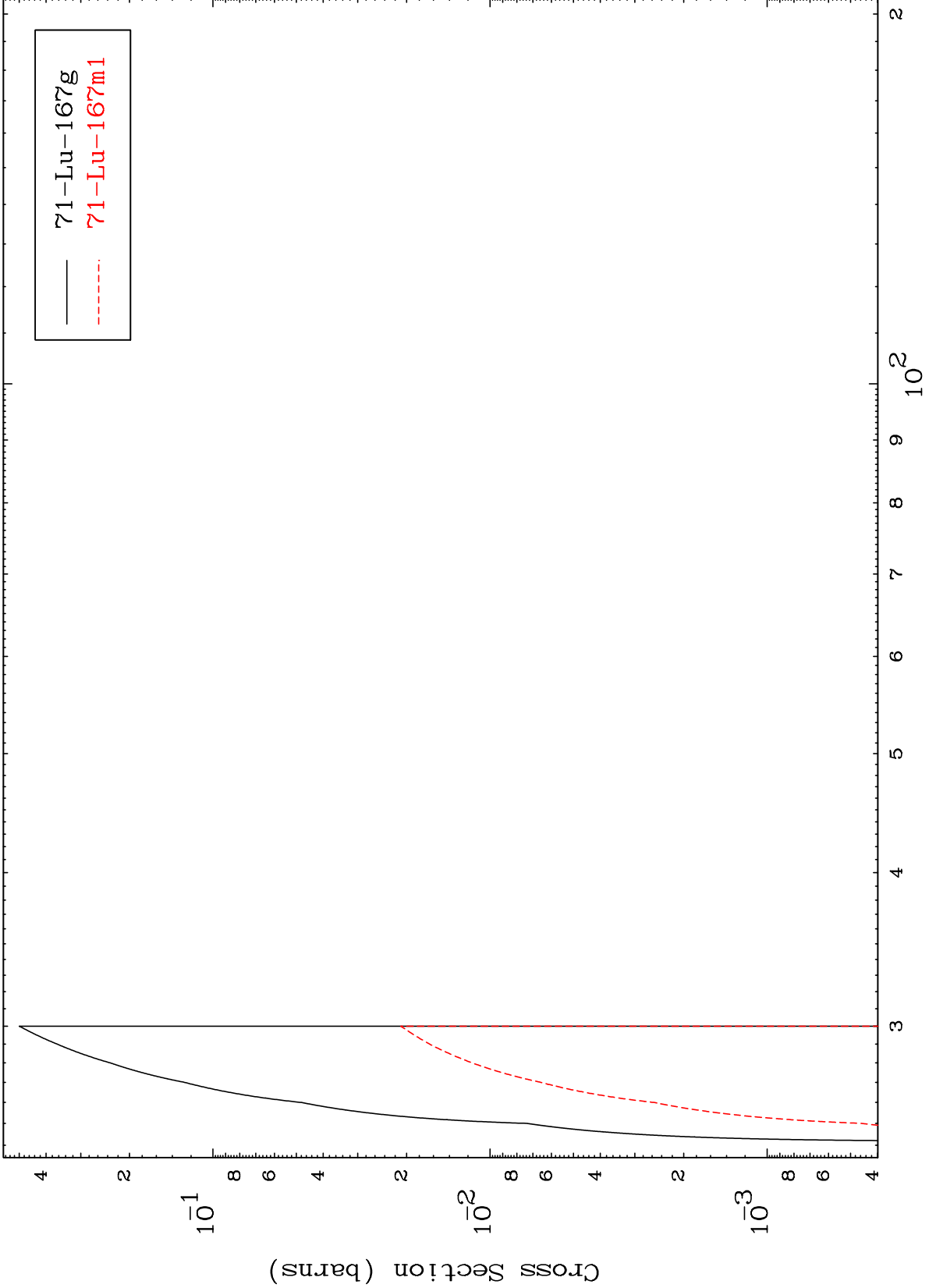


131

Incident Energy (MeV)

$^{71}\text{Lu-170}$

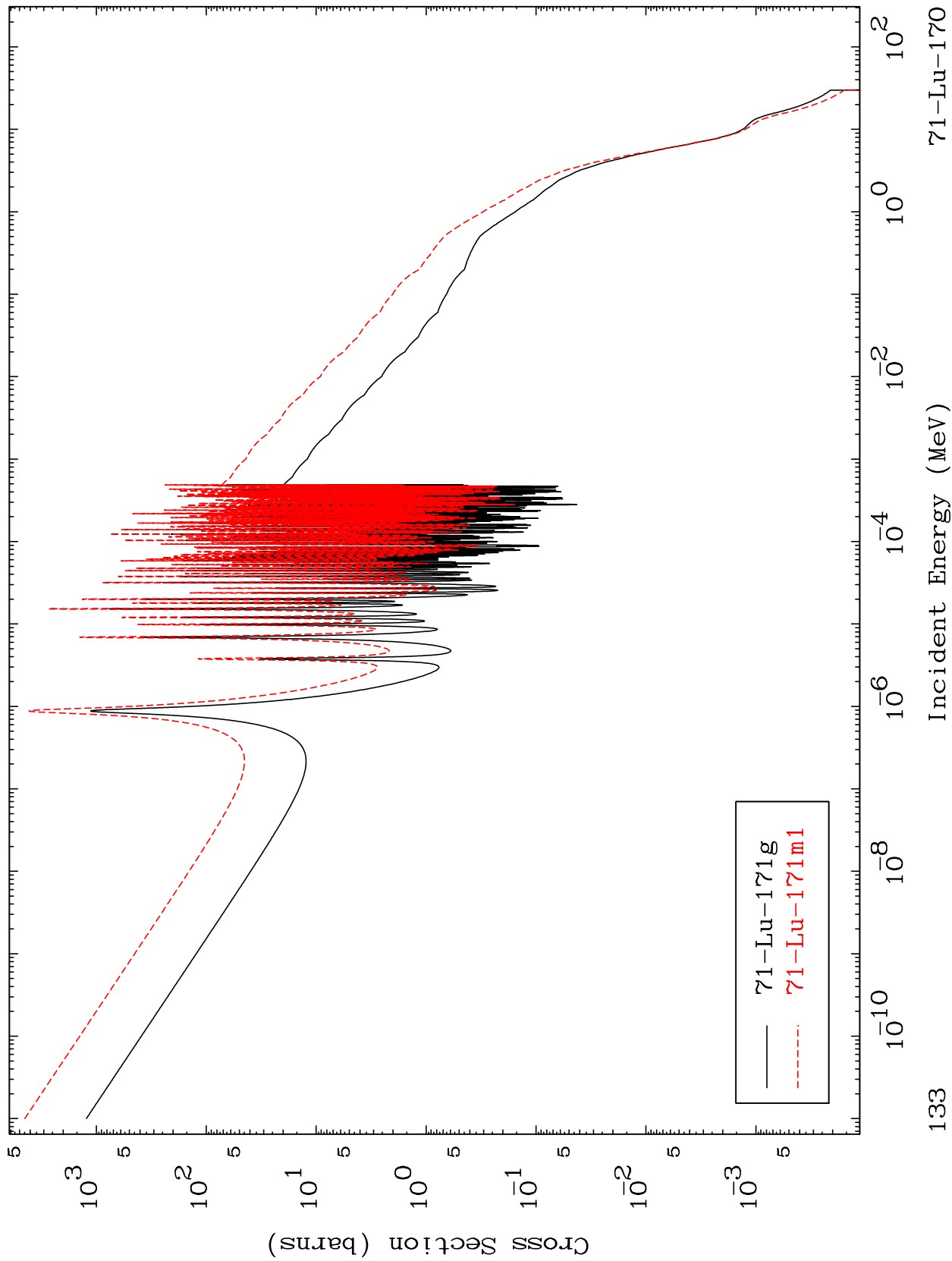
Radionuclide Production Cross Section



MAT 7110

<sup>71</sup>Lu-170

(n,γ)  
Radionuclide Production Cross Section

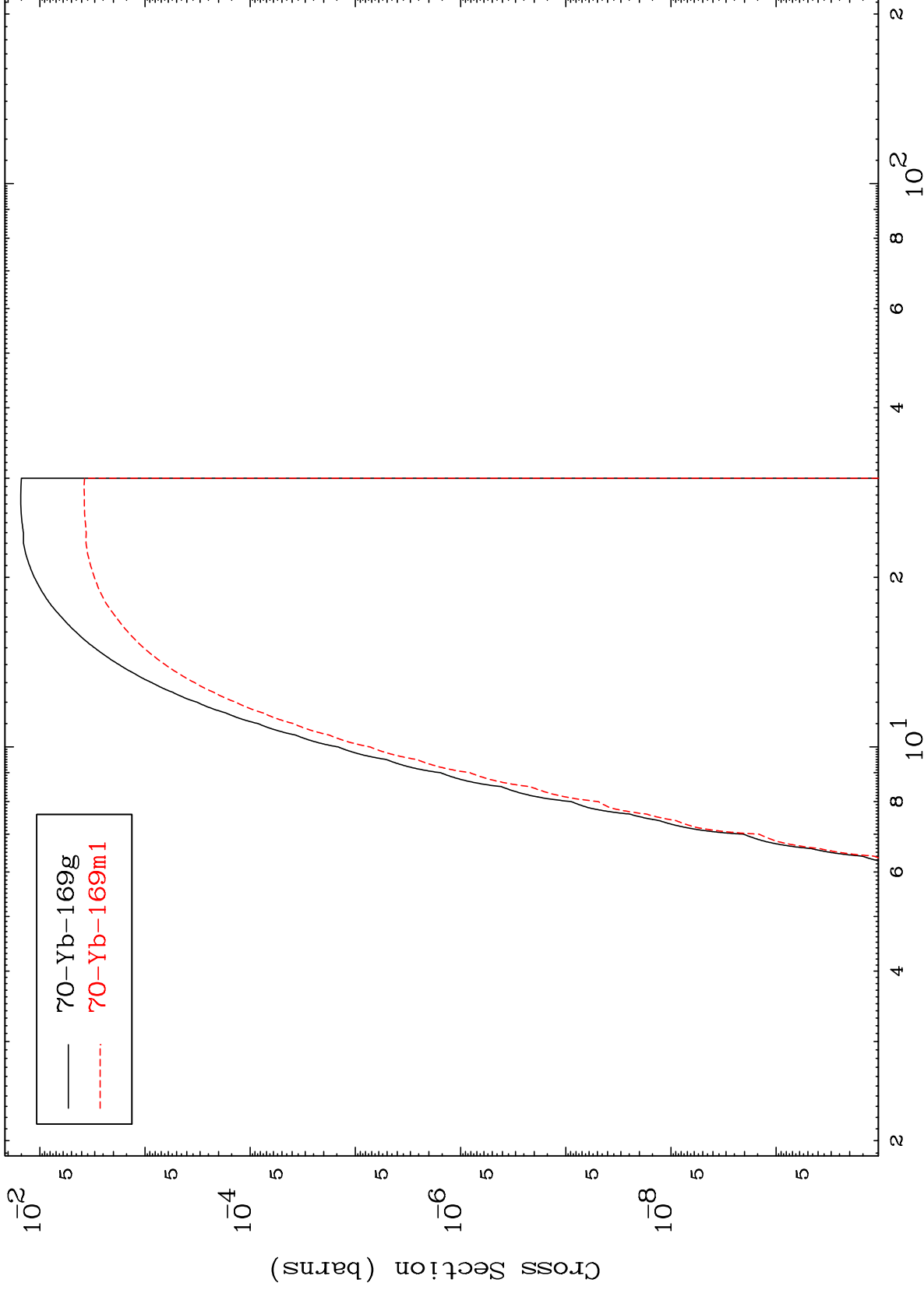


MAT 7110

(n,d)

<sup>71</sup>Lu-170

Radionuclide Production Cross Section



134

Incident Energy (MeV)

<sup>71</sup>Lu-170

MAT 7110

(n,2α)

<sup>71</sup>Lu-170

Radionuclide Production Cross Section

