

Program EVALPLOT  
(Version 2015-2)

by

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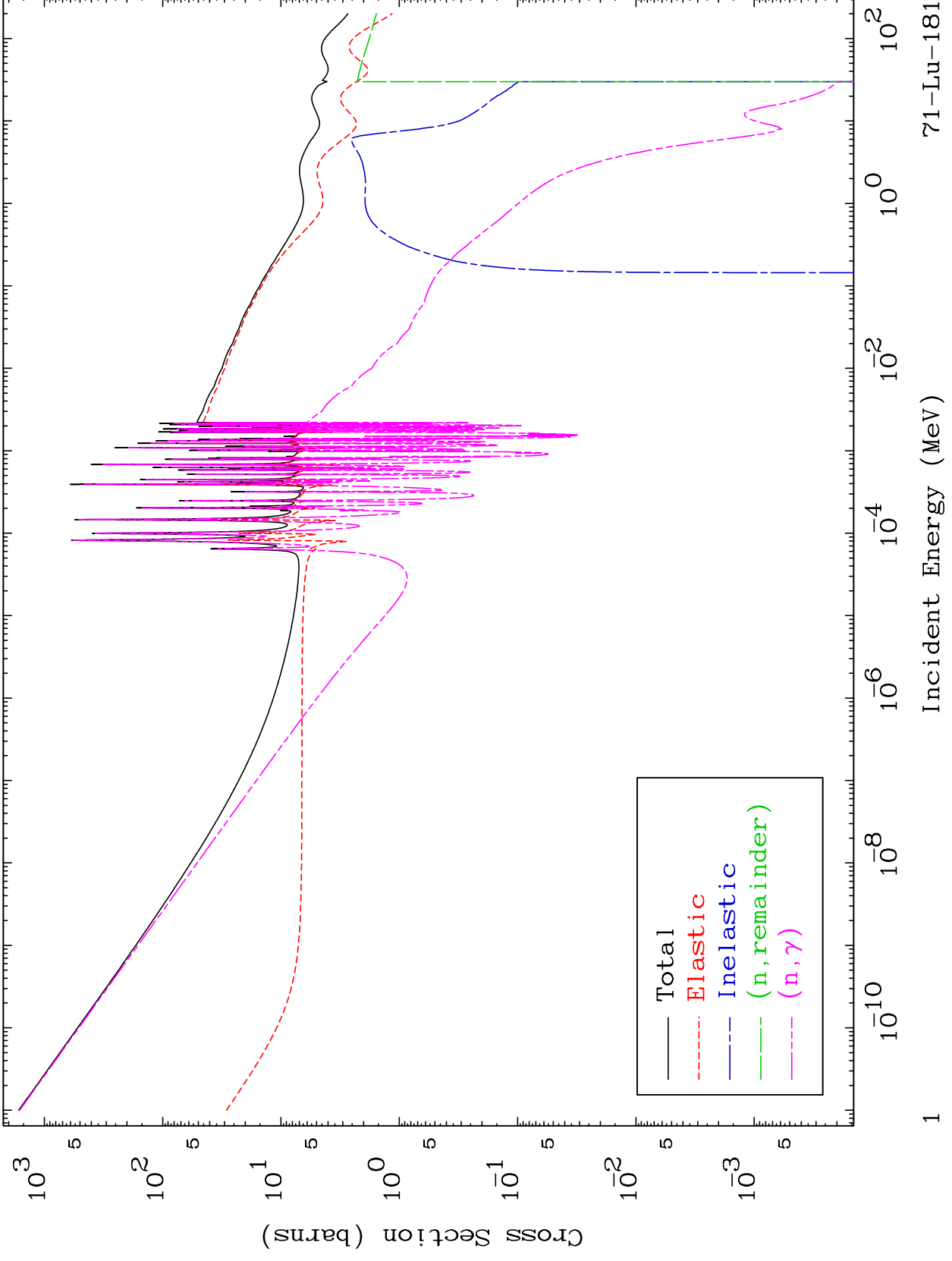
Press Mouse Button to Start

MAT 7143

Major

293 Kelvin Cross Sections

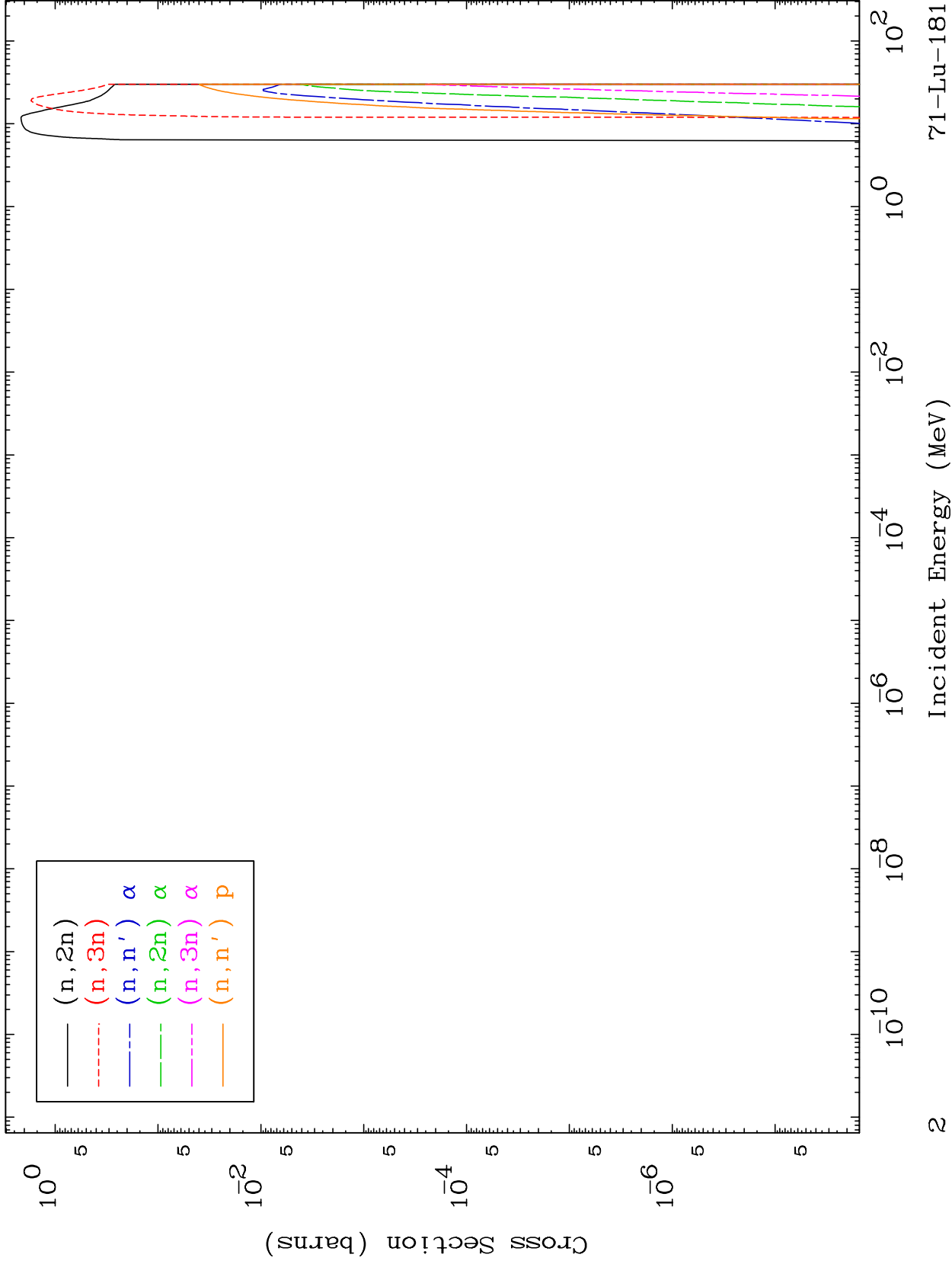
71-Lu-181

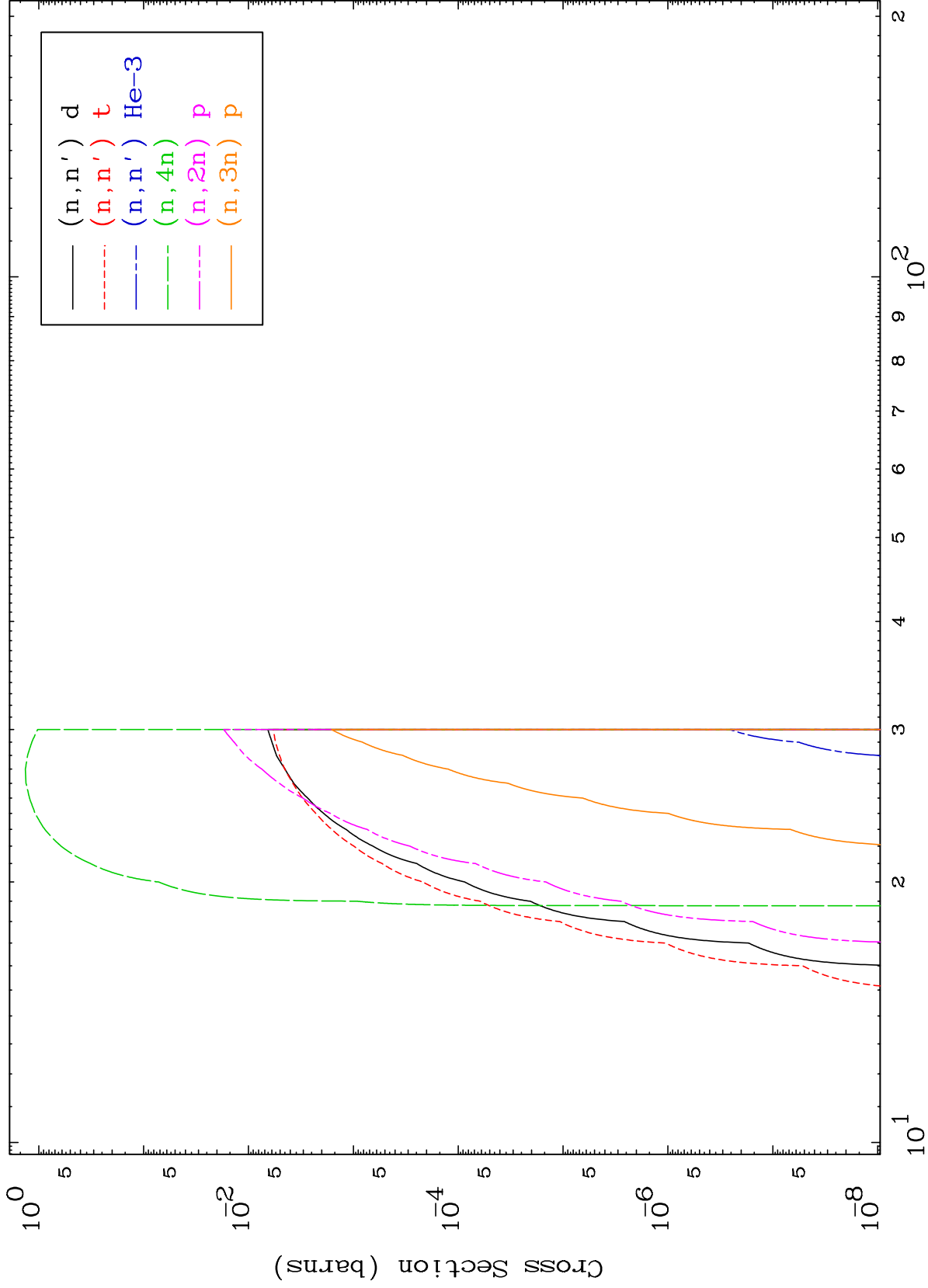


MAT 7143

Neutron Production  
293 Kelvin Cross Sections

71-Lu-181

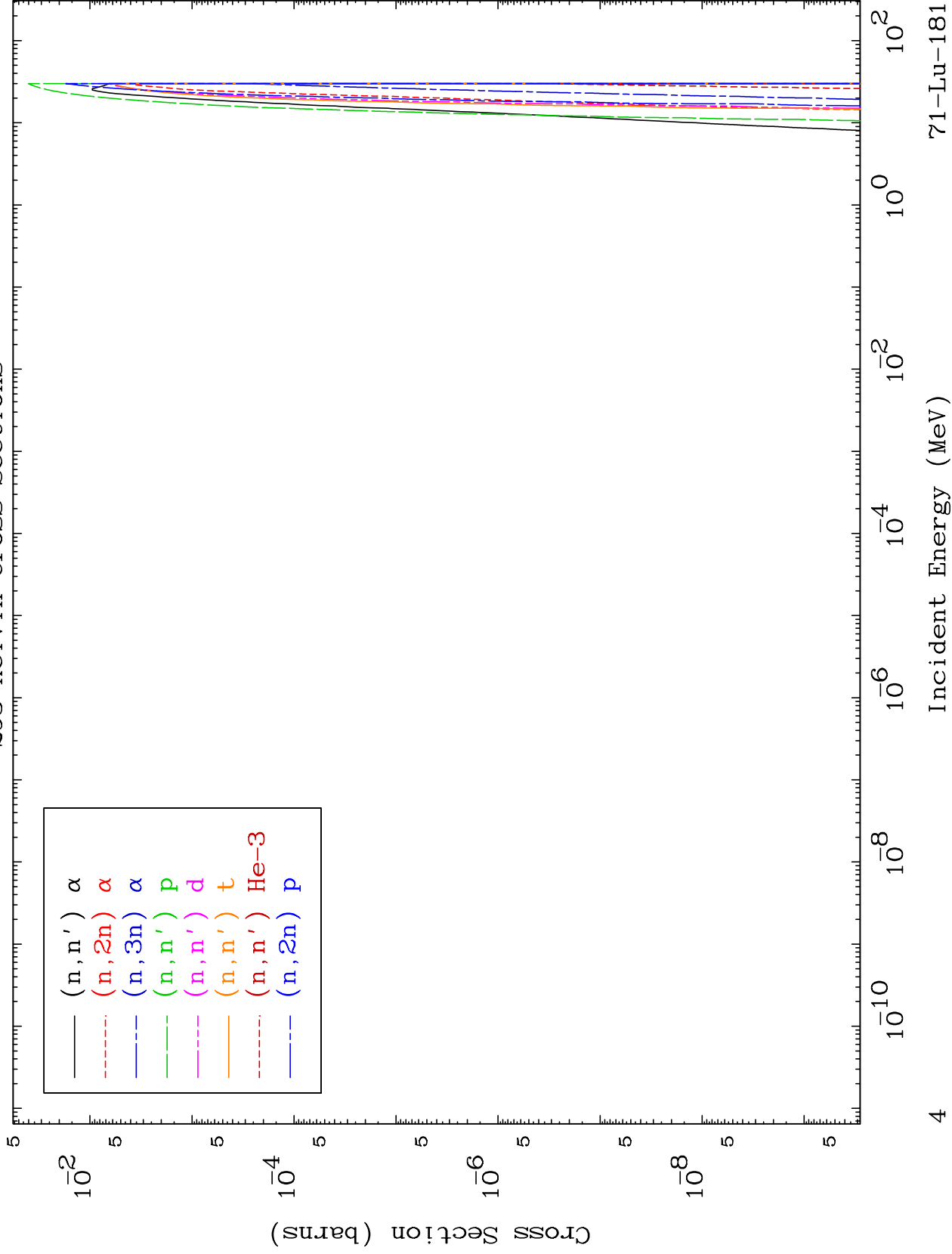




MAT 7143

Charged Particle  
293 Kelvin Cross Sections

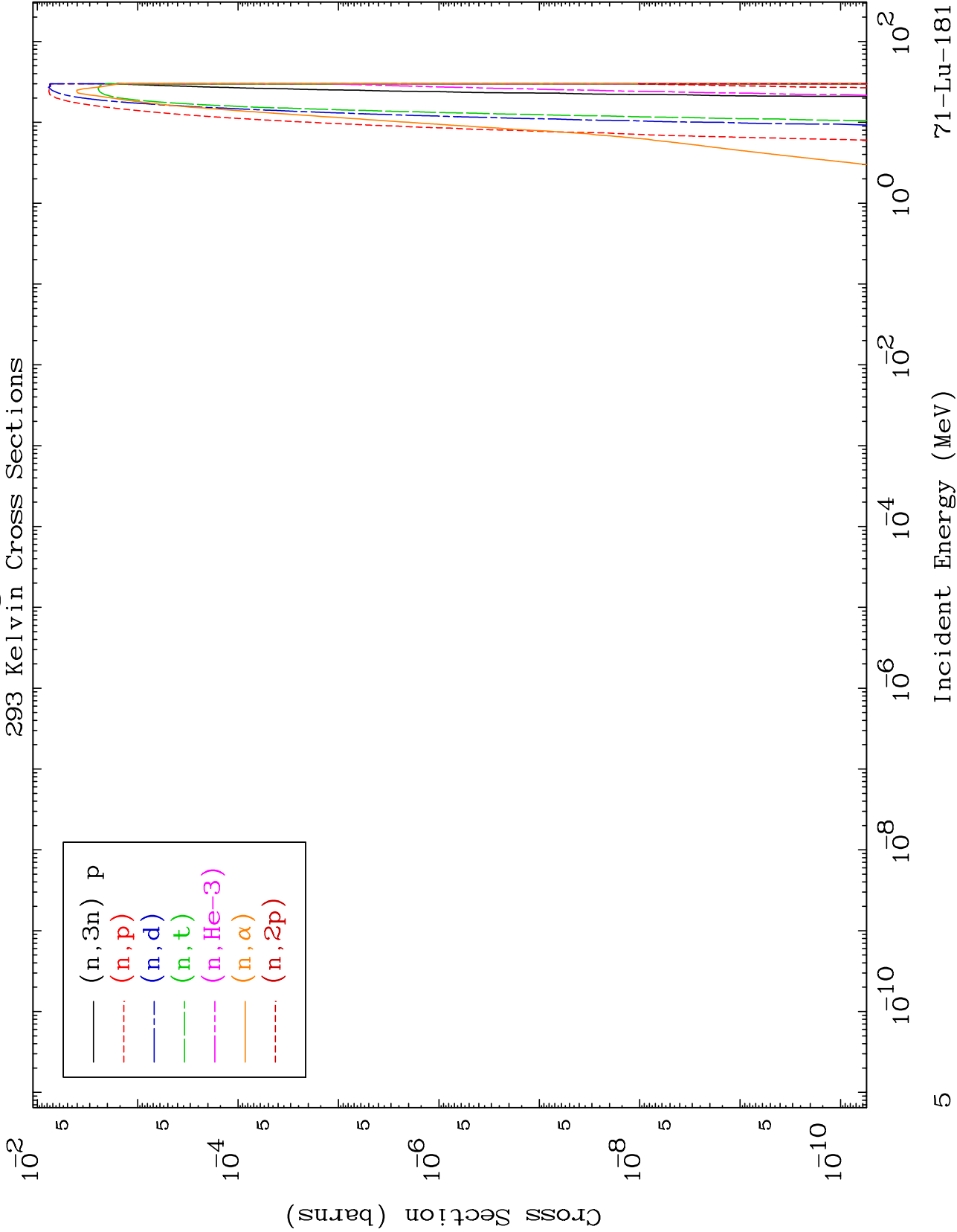
71-Lu-181



MAT 7143

Charged Particle  
293 Kelvin Cross Sections

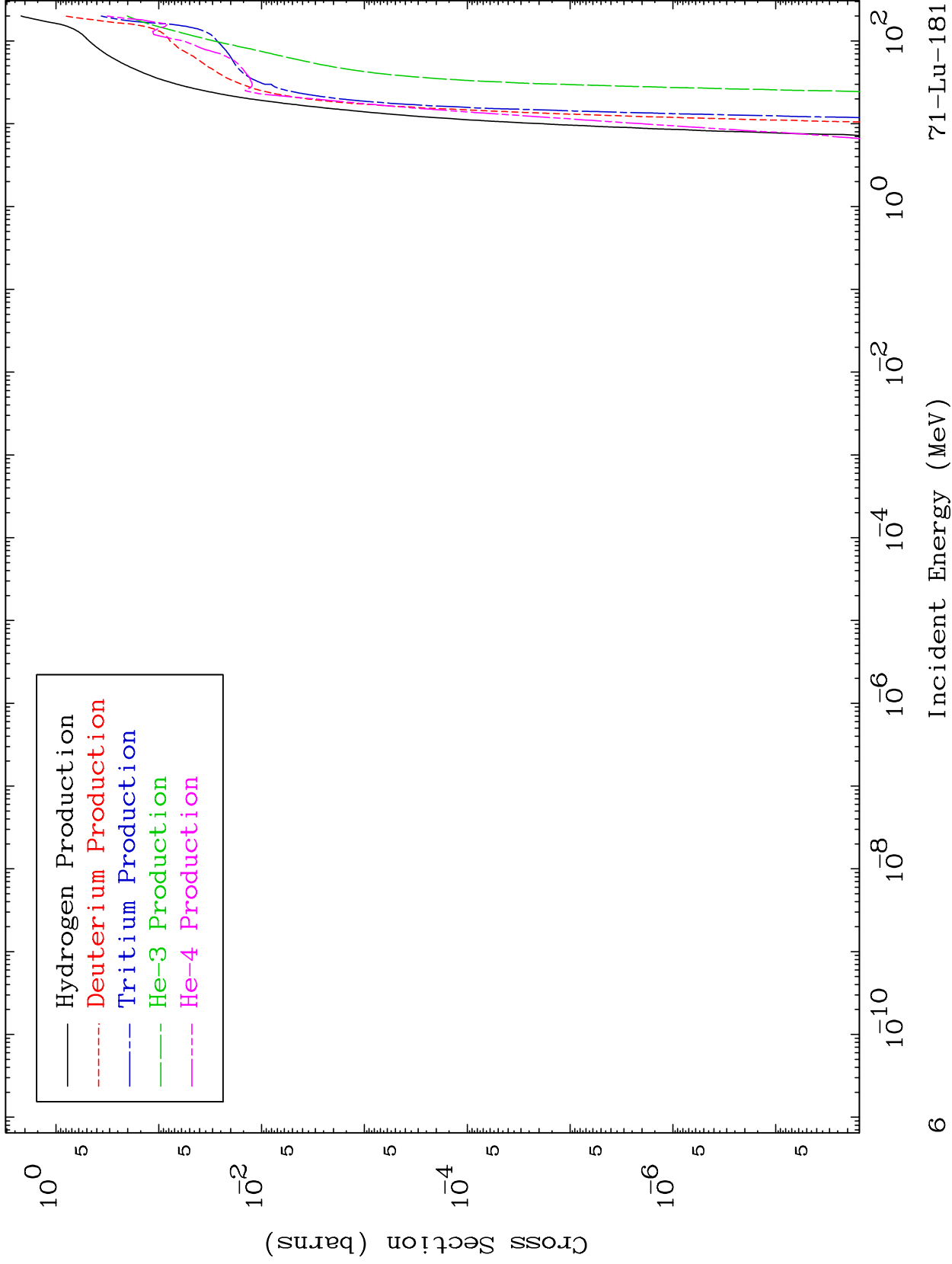
71-Lu-181



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Particle Production  
293 Kelvin Cross Sections

71-Lu-181

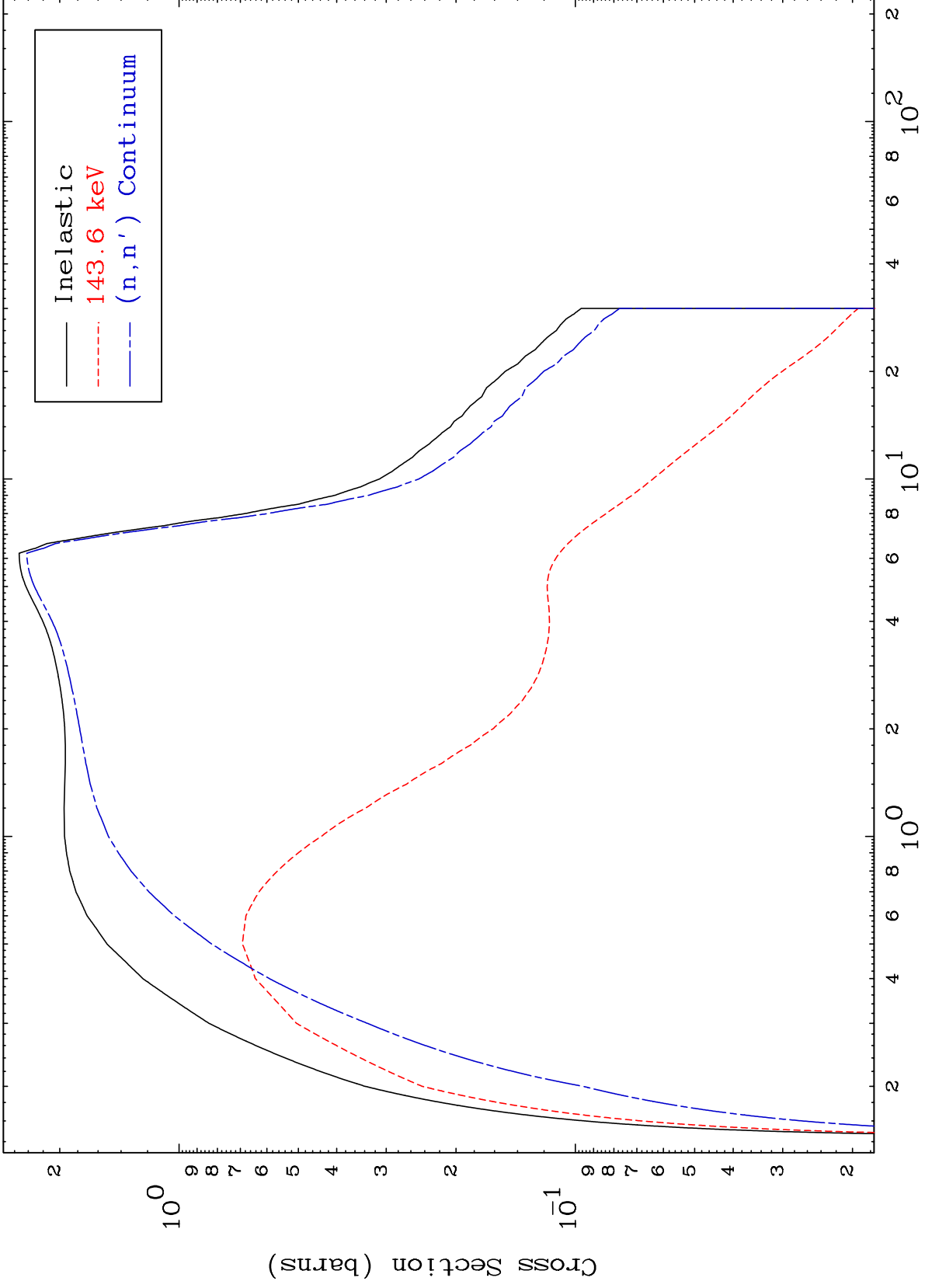


MAT 7143

(n,n') Level

71-Lu-181

293 Kelvin Cross Sections



7

Incident Energy (MeV)

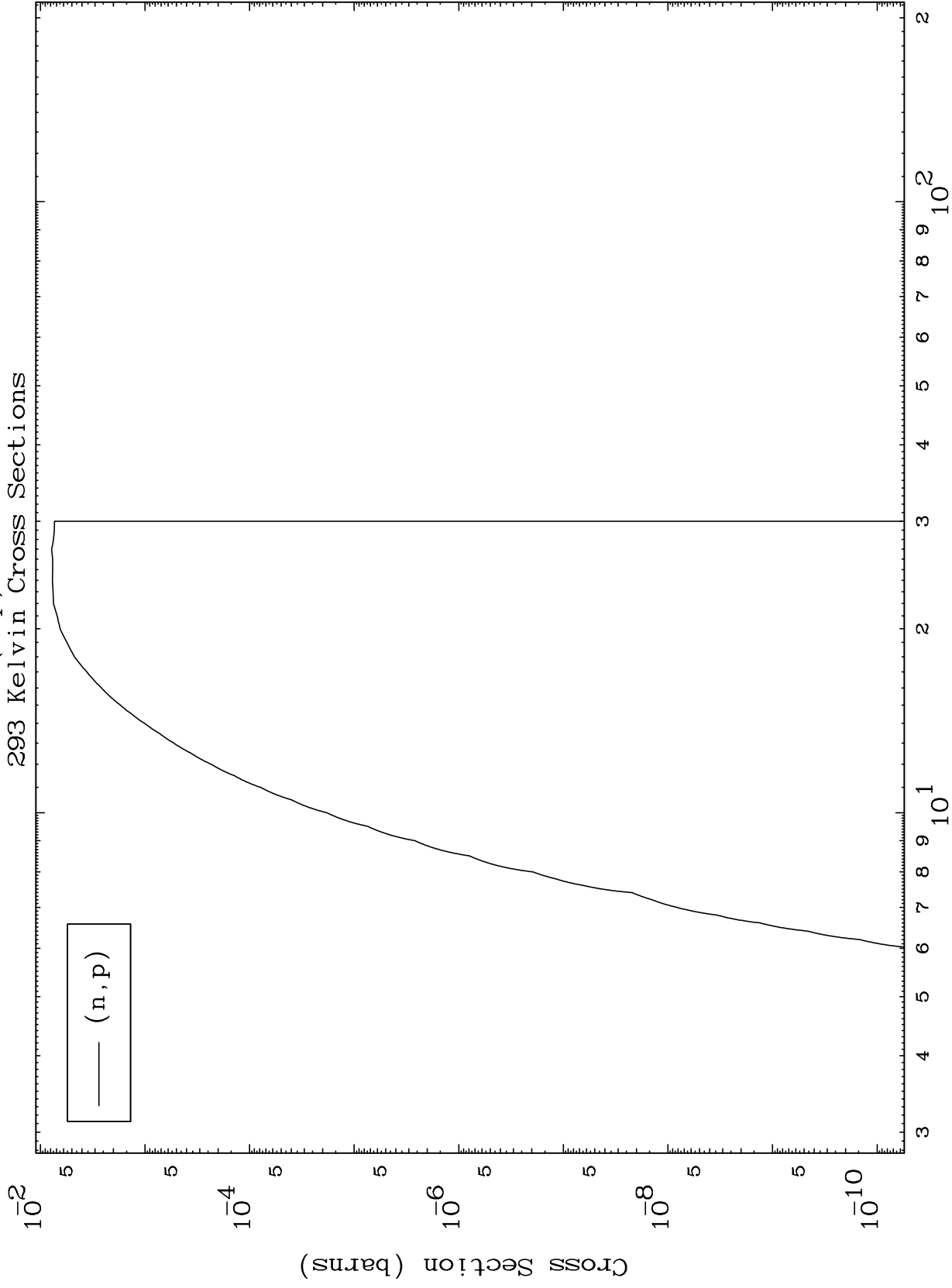
71-Lu-181



MAT 7143

(n,p) Levels  
293 Kelvin Cross Sections

71-Lu-181



8

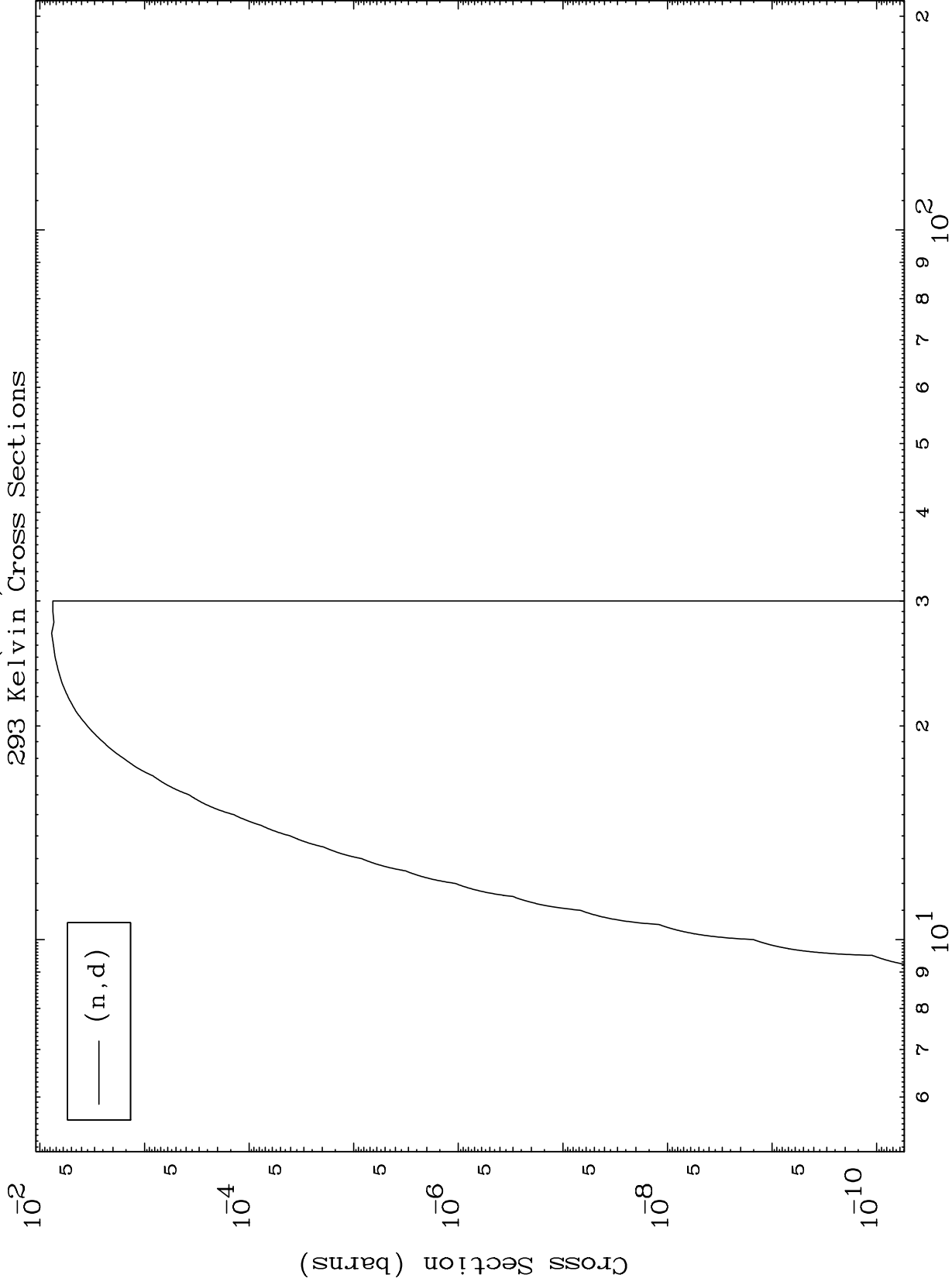
Incident Energy (MeV)

71-Lu-181

MAT 7143

71-Lu-181

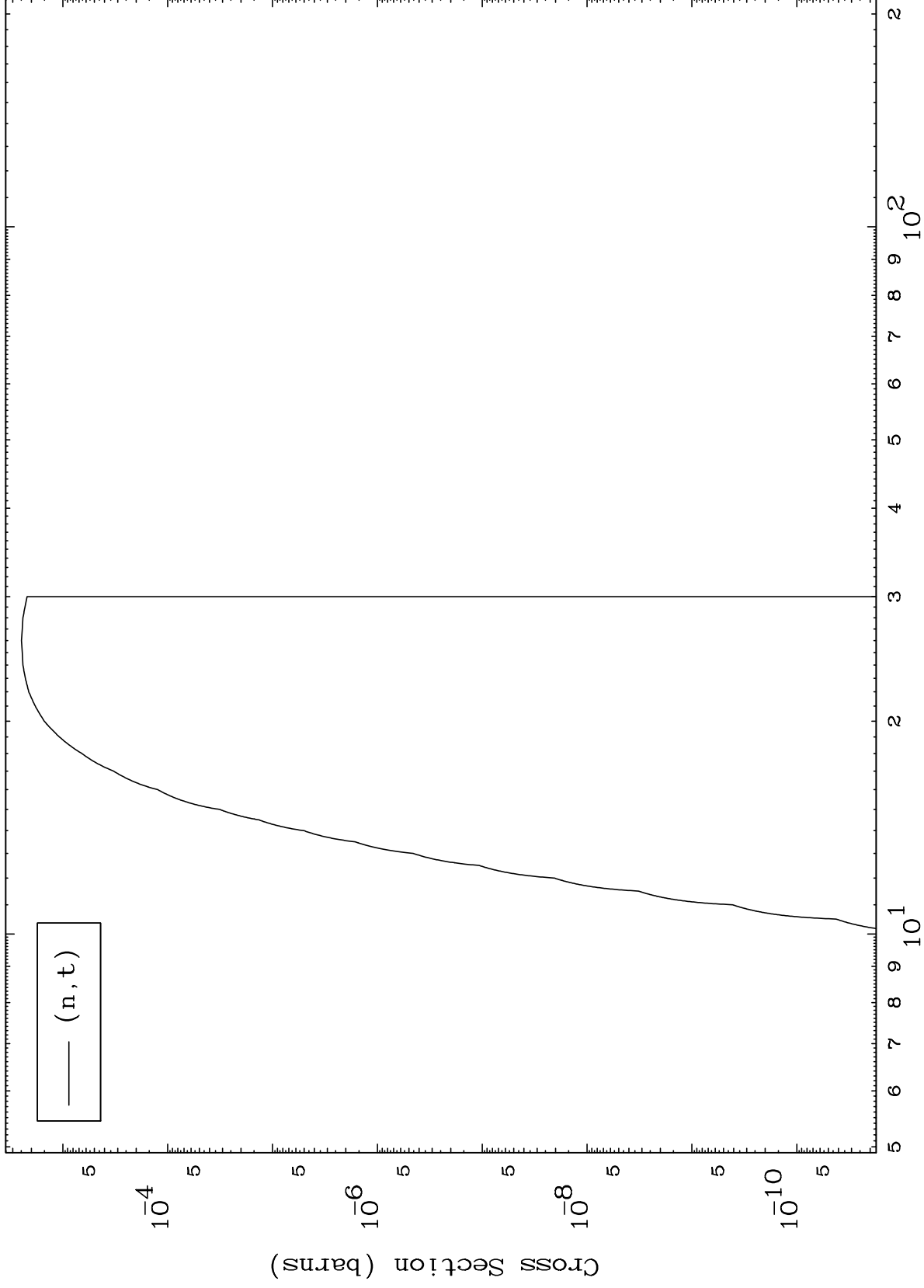
(n,d) Levels  
293 Kelvin Cross Sections



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(n,t) Levels  
293 Kelvin Cross Sections

71-Lu-181



10

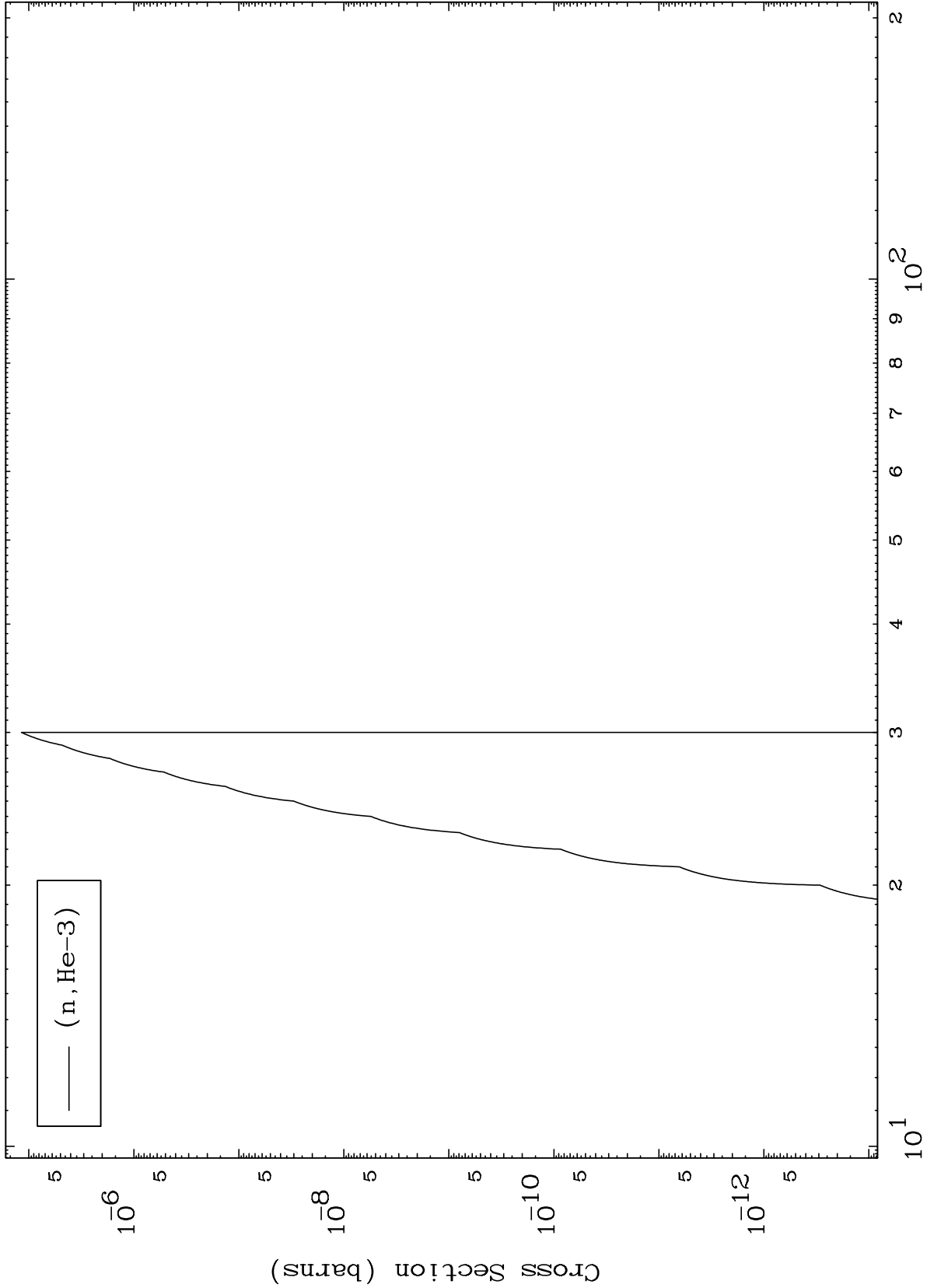
Incident Energy (MeV)

71-Lu-181

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71-Lu-181

(n,He3) Levels  
293 Kelvin Cross Sections



71-Lu-181

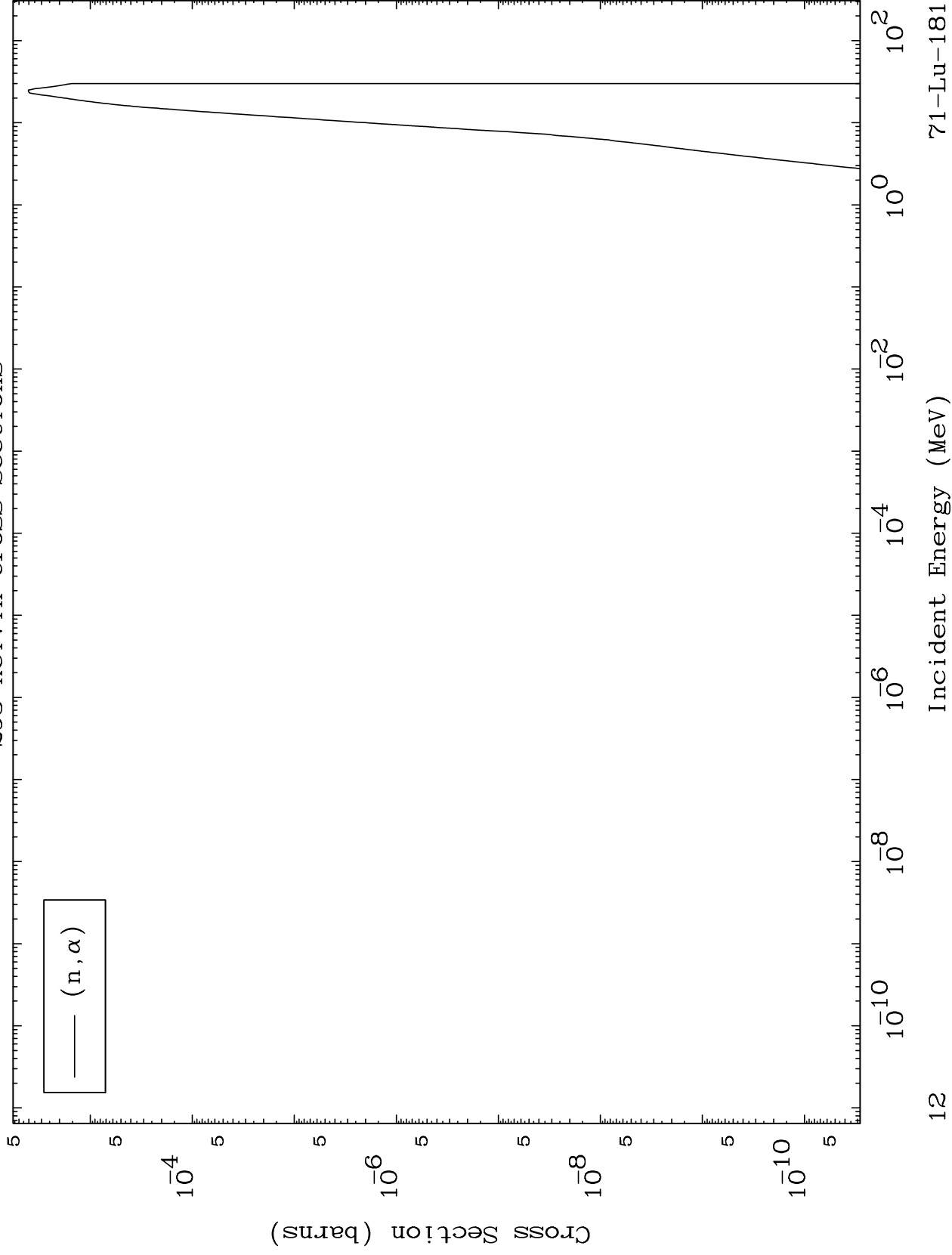
Incident Energy (MeV)

11

MAT 7143

(n,  $\alpha$ ) Levels  
293 Kelvin Cross Sections

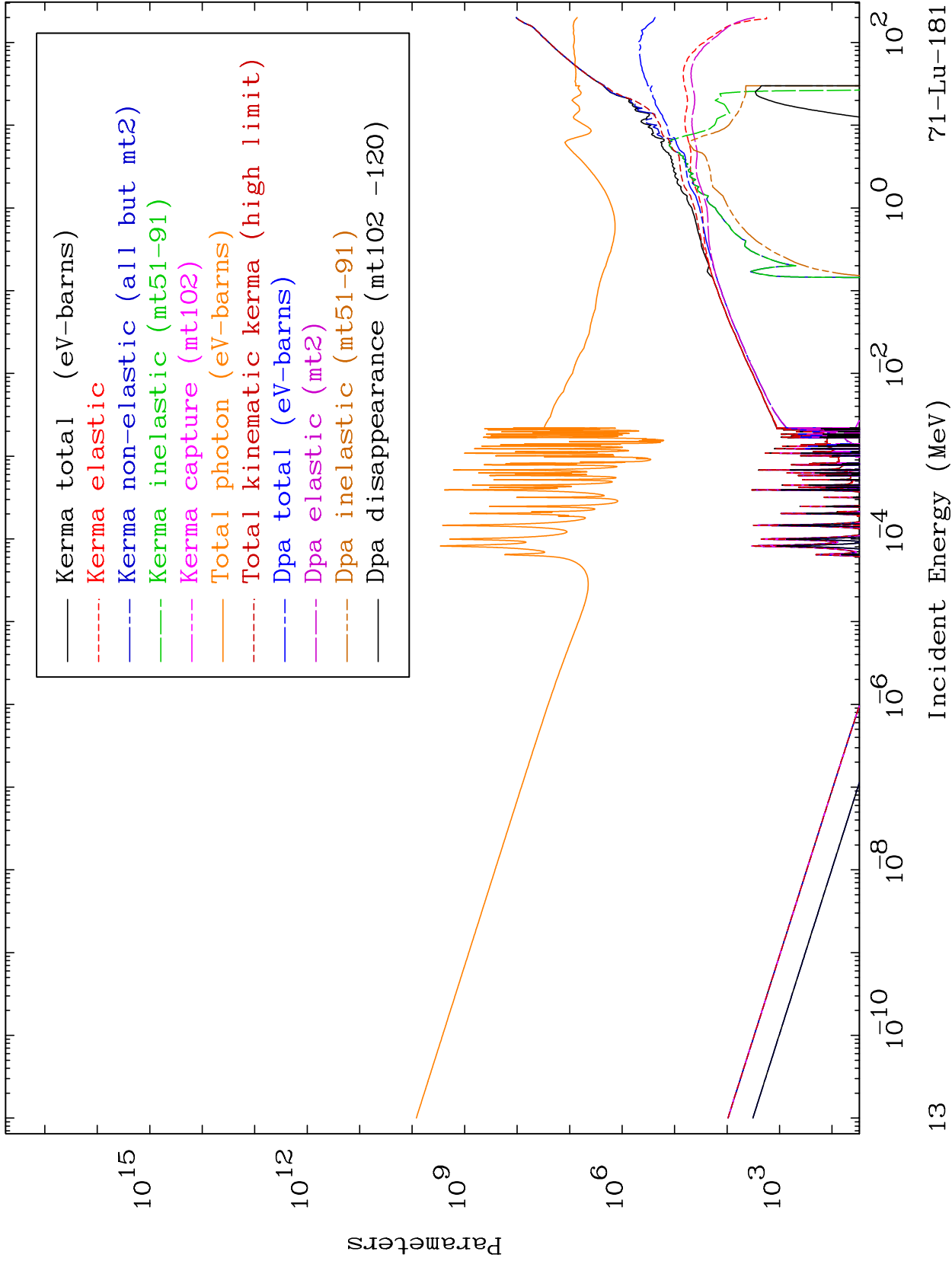
71-Lu-181



12

Incident Energy (MeV)

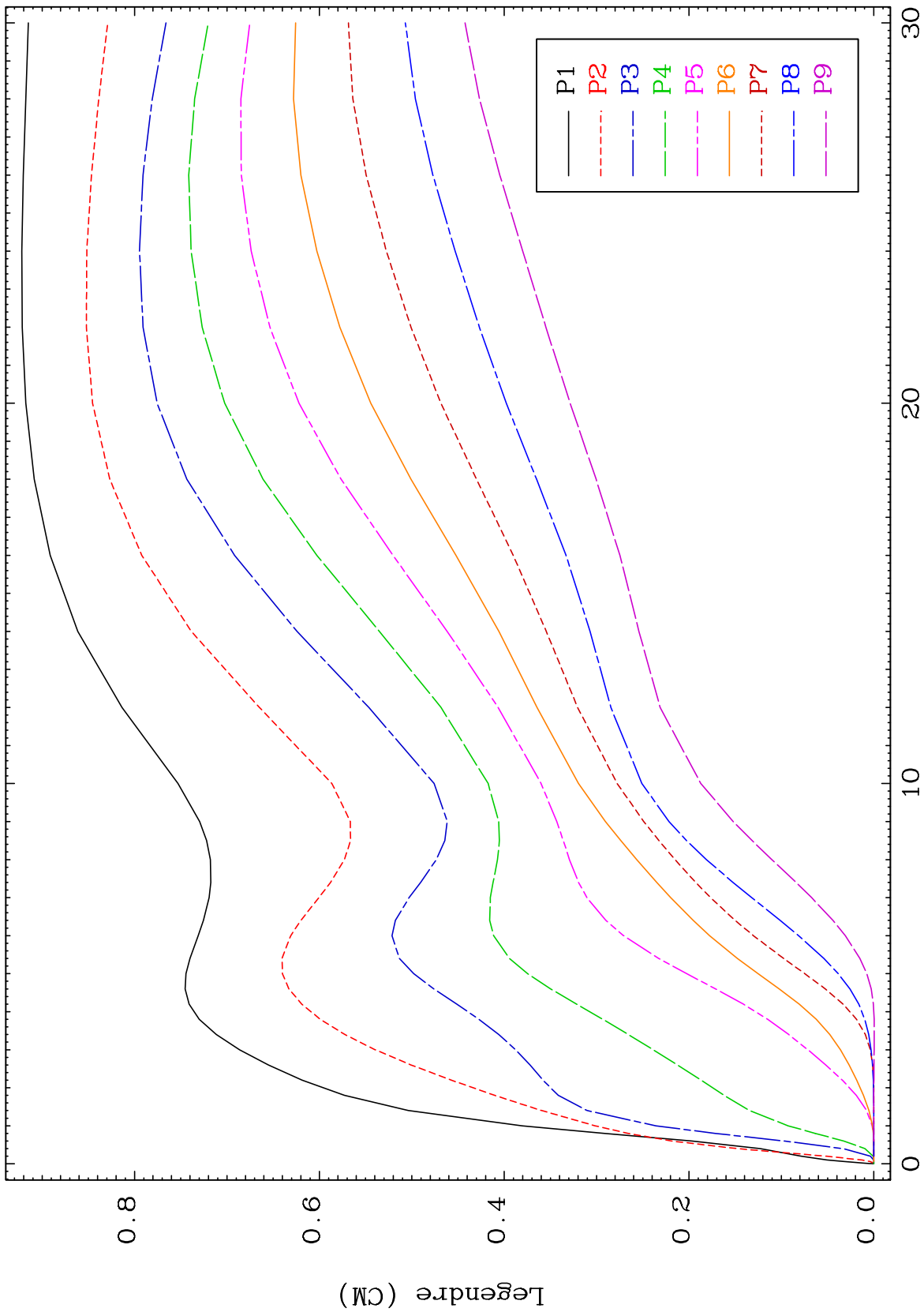
71-Lu-181



MAT 7143

71-Lu-181

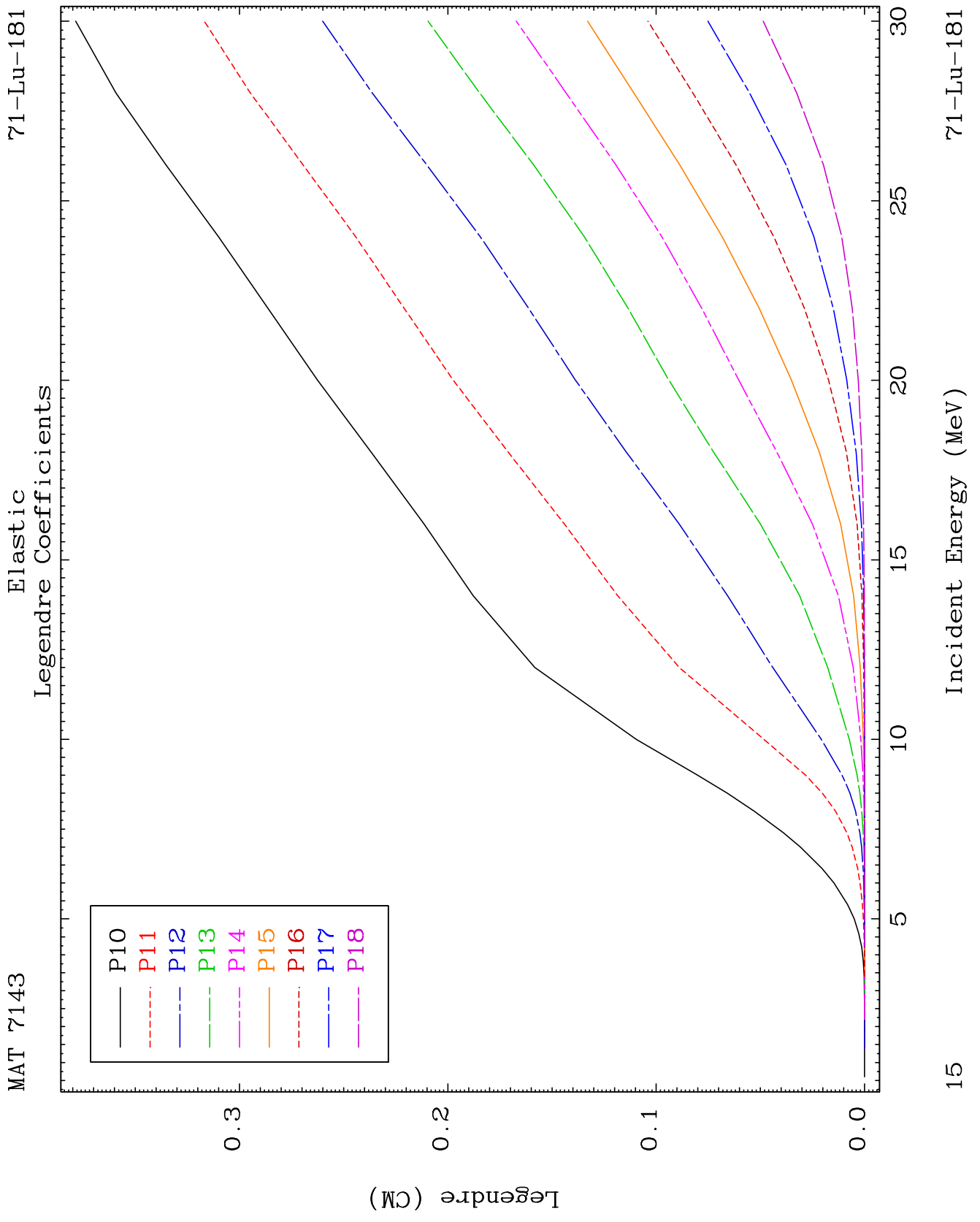
Elastic Legendre Coefficients



14

Incident Energy (MeV)

71-Lu-181

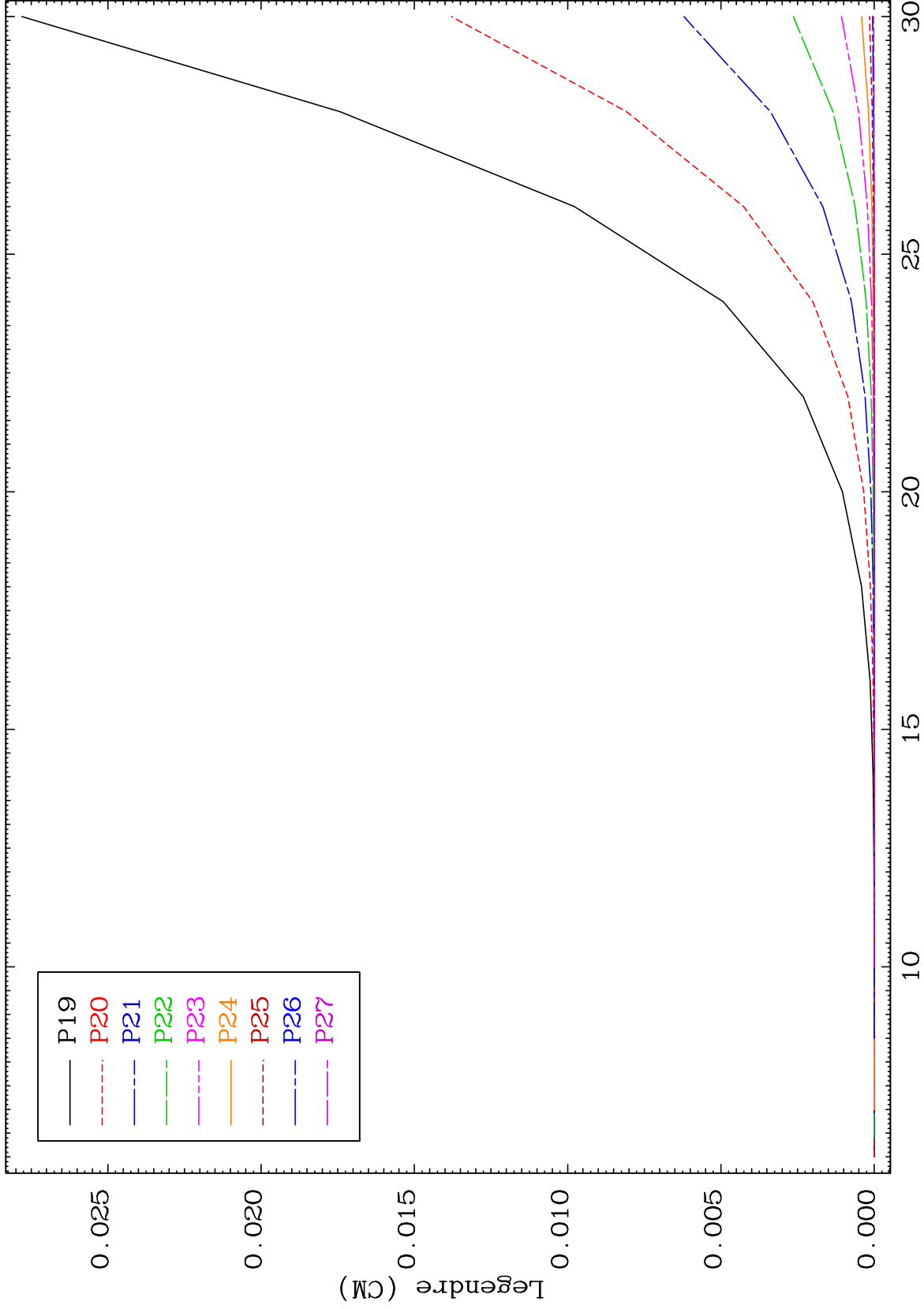




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Elastic  
Legendre Coefficients

71-Lu-181



16

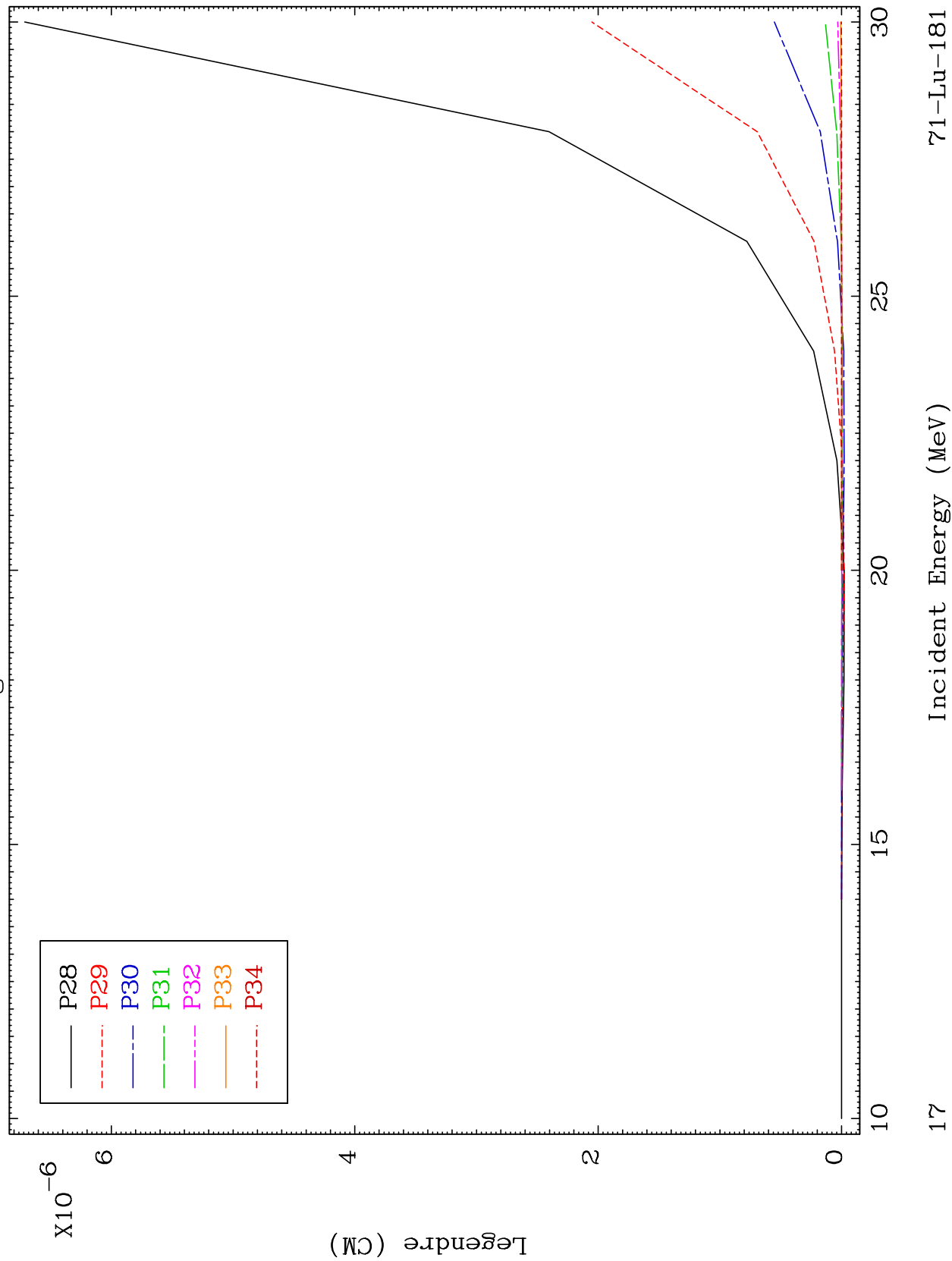
Incident Energy (MeV)

71-Lu-181

MAT 7143

Elastic  
Legendre Coefficients

71-Lu-181



17

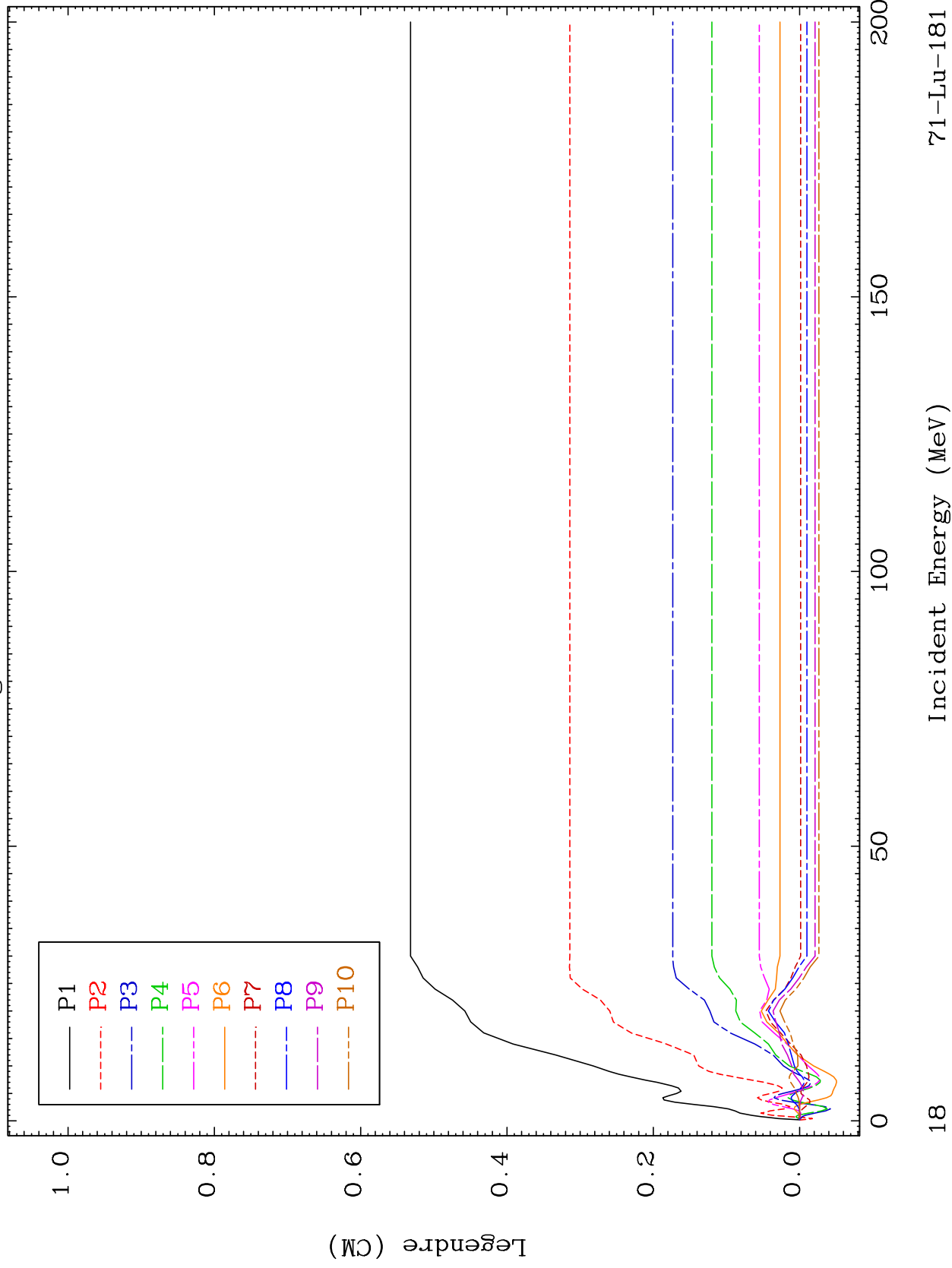
Incident Energy (MeV)

71-Lu-181

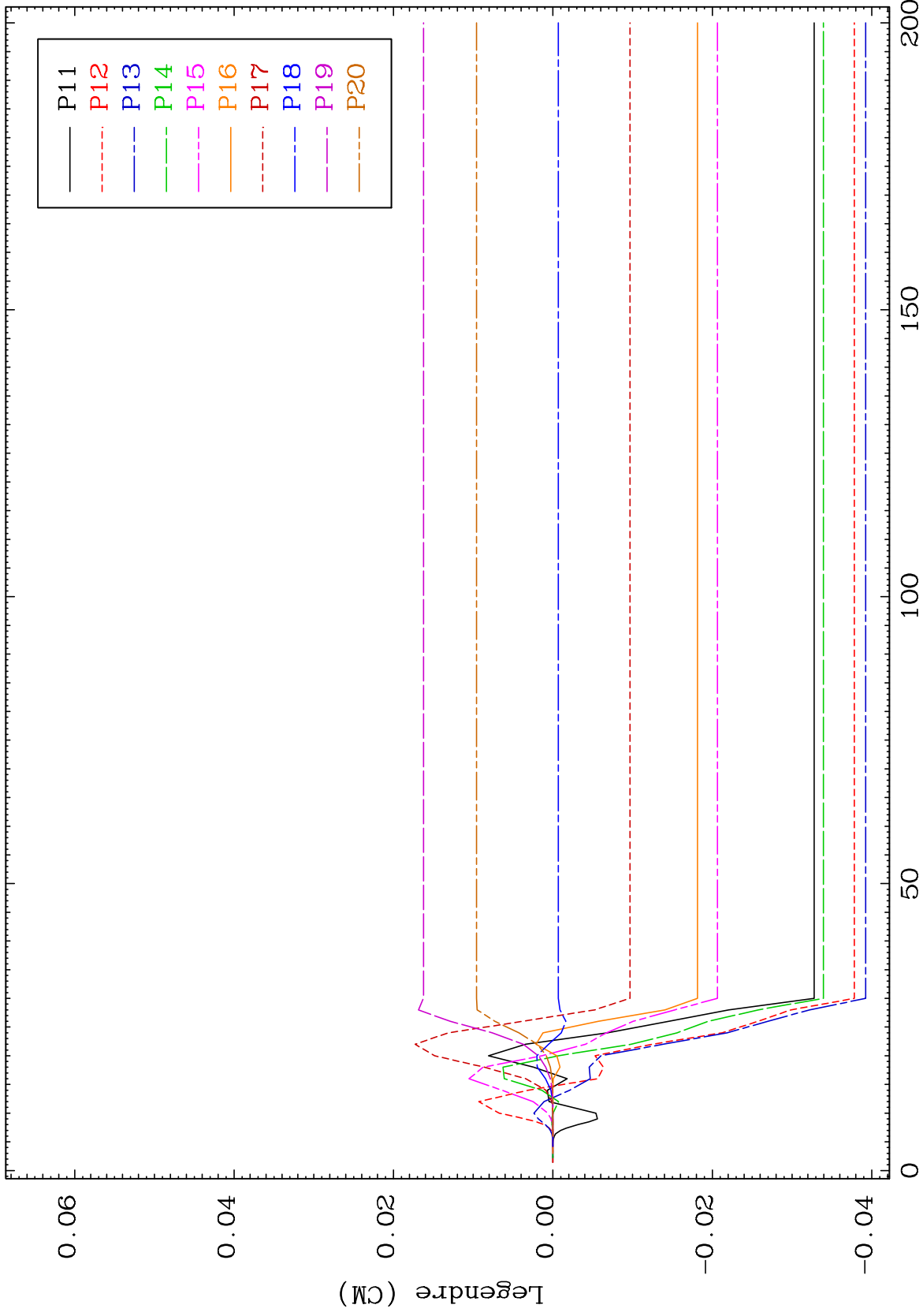
MAT 7143

143.6 keV (n, n') Level  
Legendre Coefficients

71-Lu-181



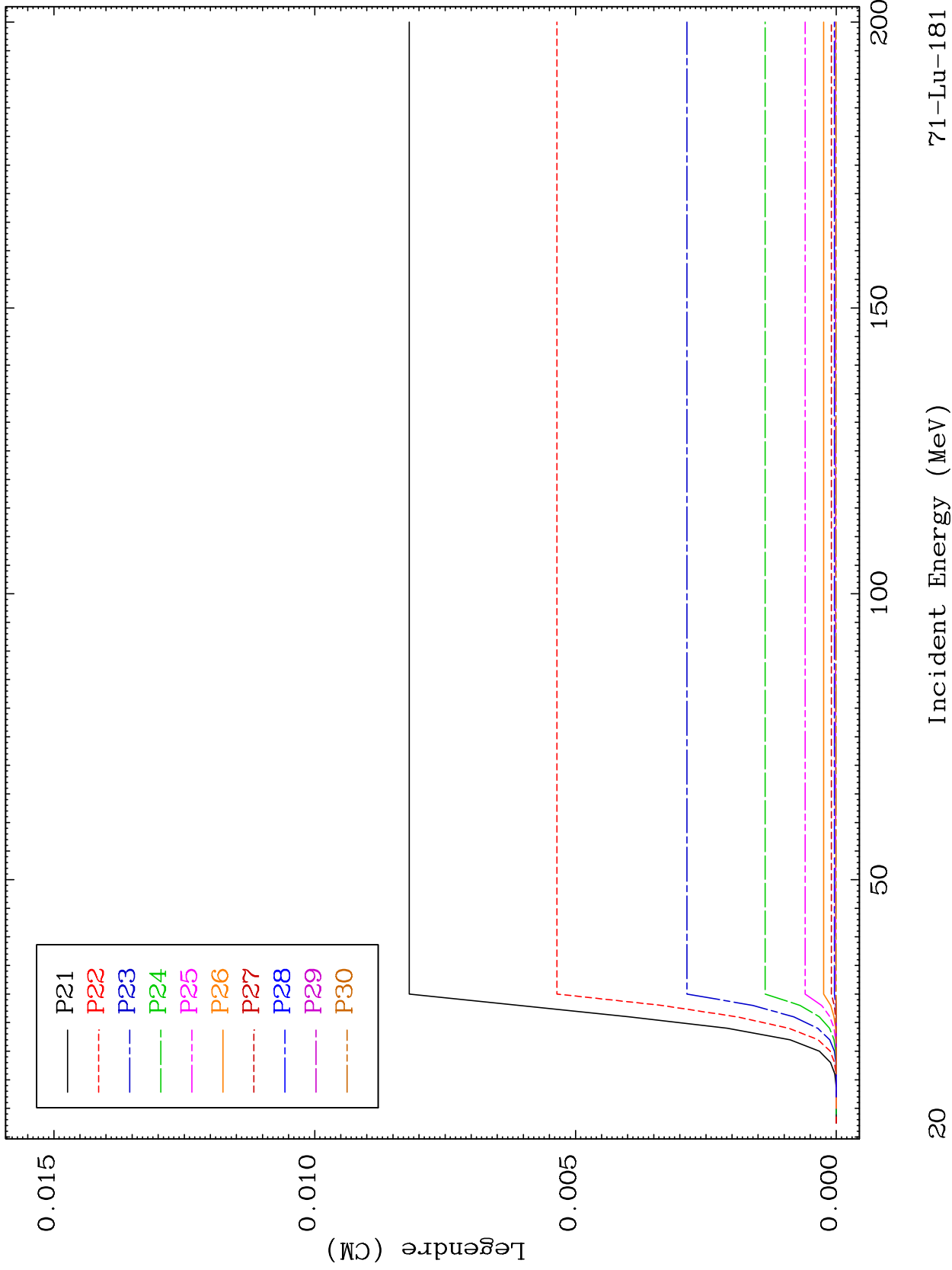
18



MAT 7143

143.6 keV (n,n') Level  
Legendre Coefficients

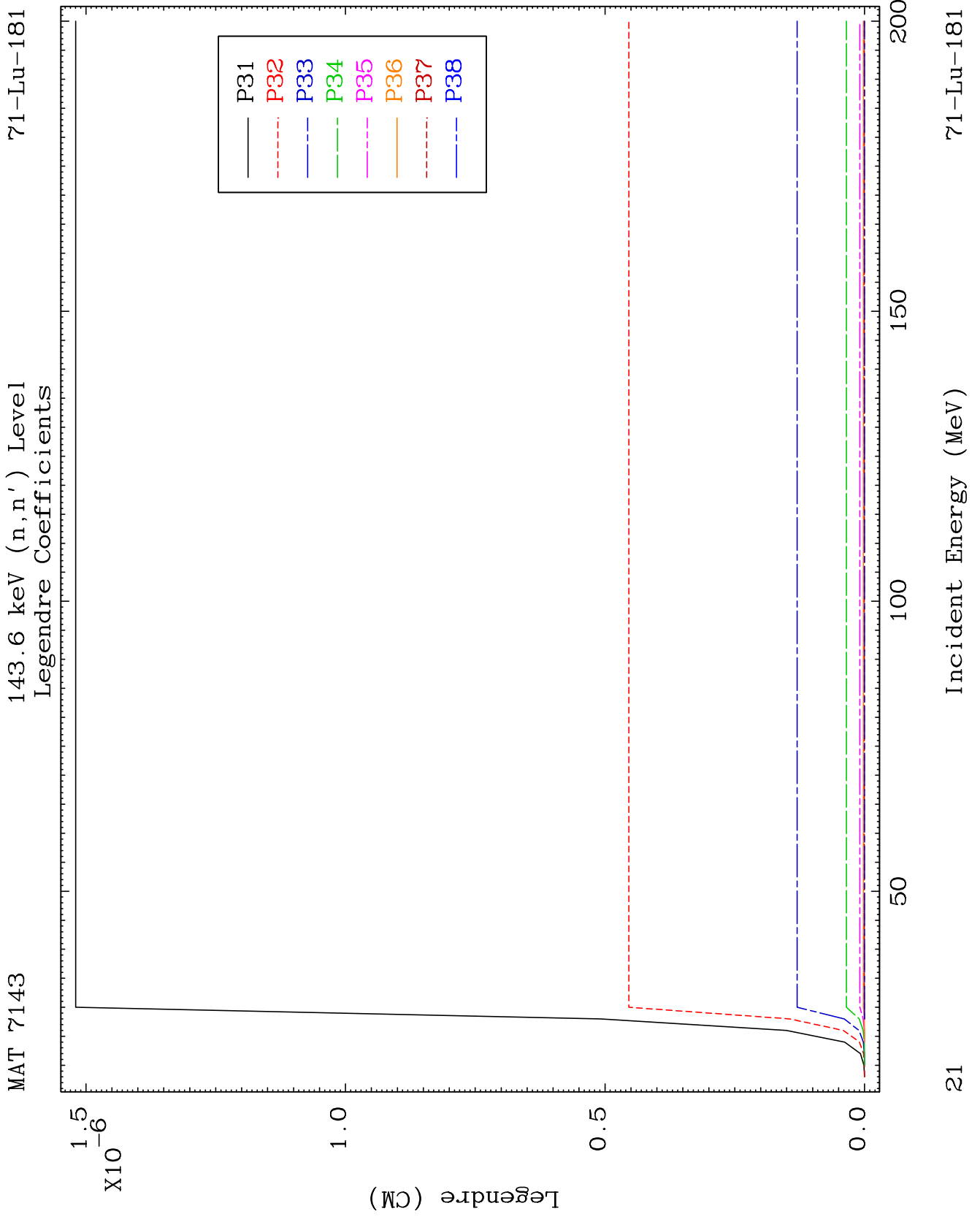
71-Lu-181



20

Incident Energy (MeV)

71-Lu-181

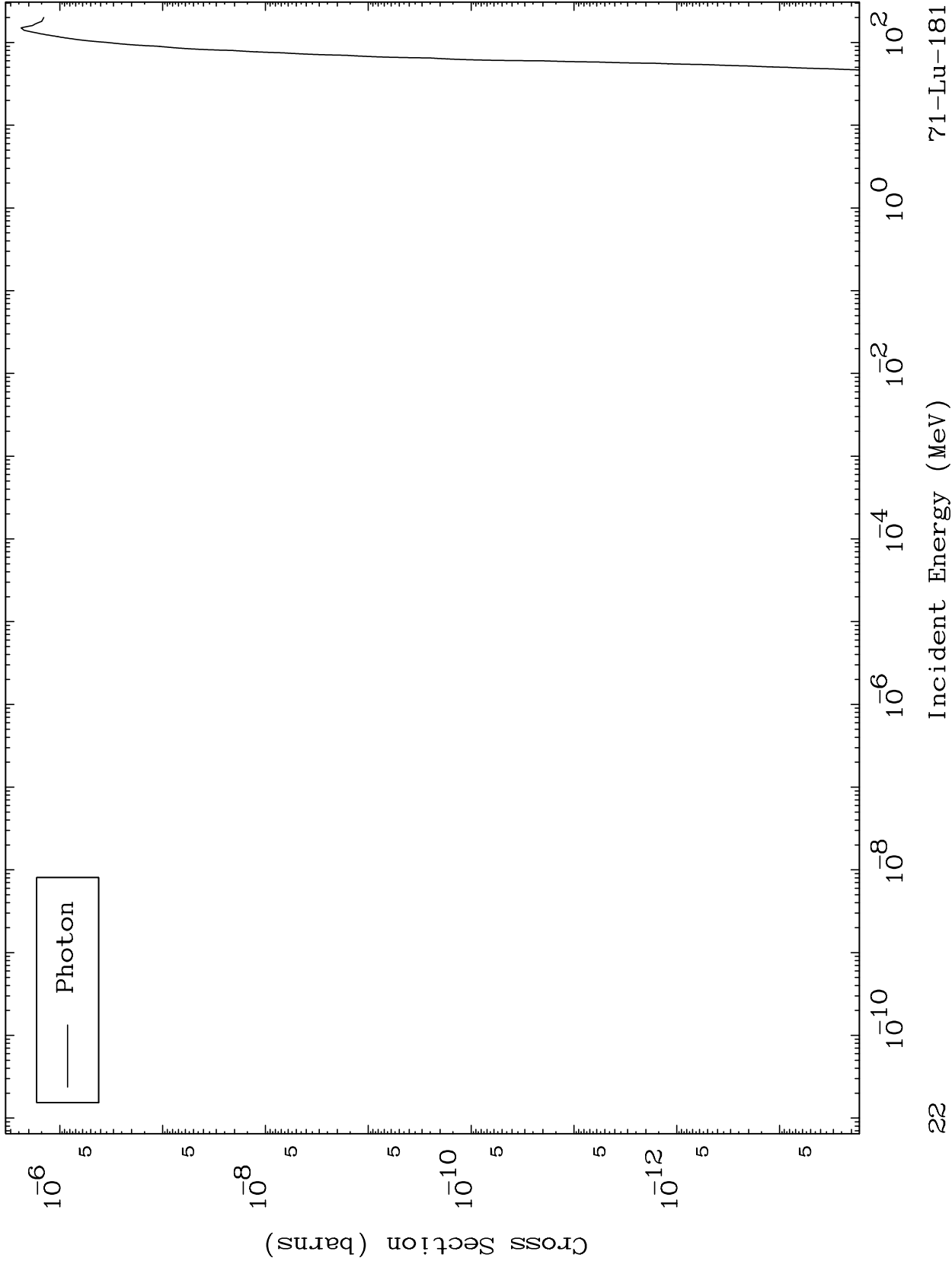


MAT 7143

Fission

<sup>71</sup>Lu-181

Radionuclide Production Cross Section

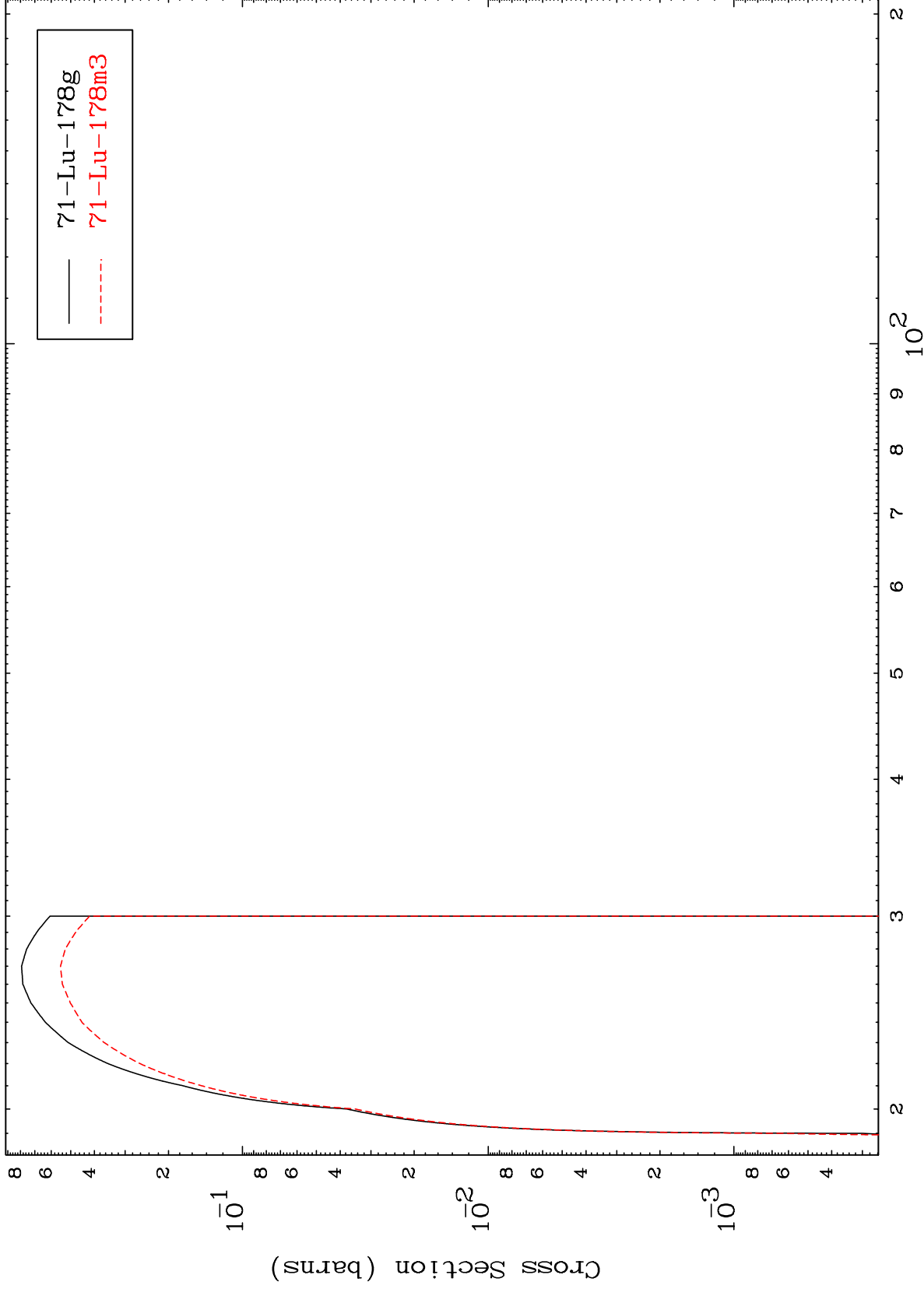


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(n,4n)

71-Lu-181

Radionuclide Production Cross Section



23

Incident Energy (MeV)

71-Lu-181