

Program EVALPLOT
(Version 2015-2)

by

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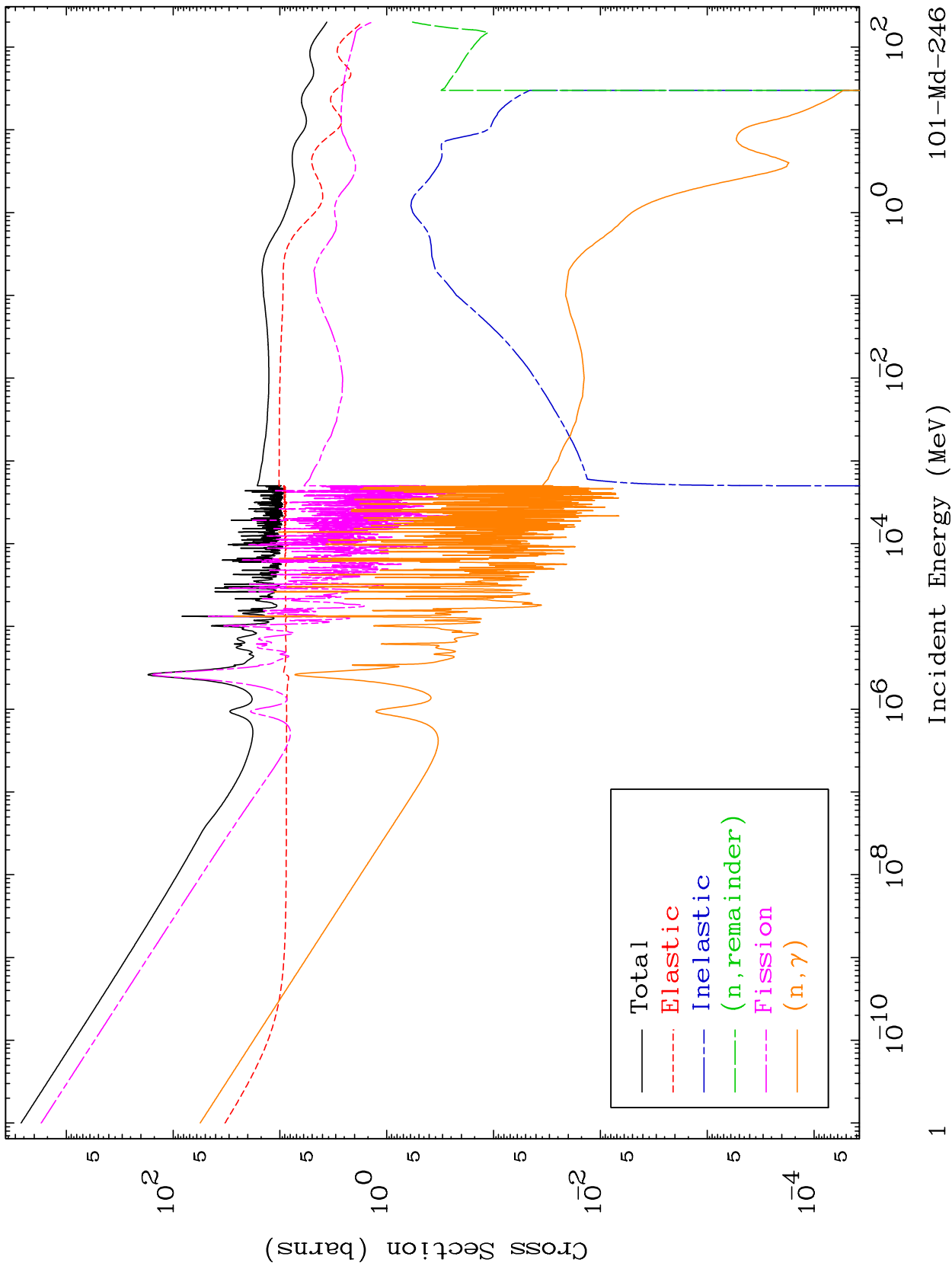
Press Mouse Button to Start

MAT 9952

Major

293 Kelvin Cross Sections

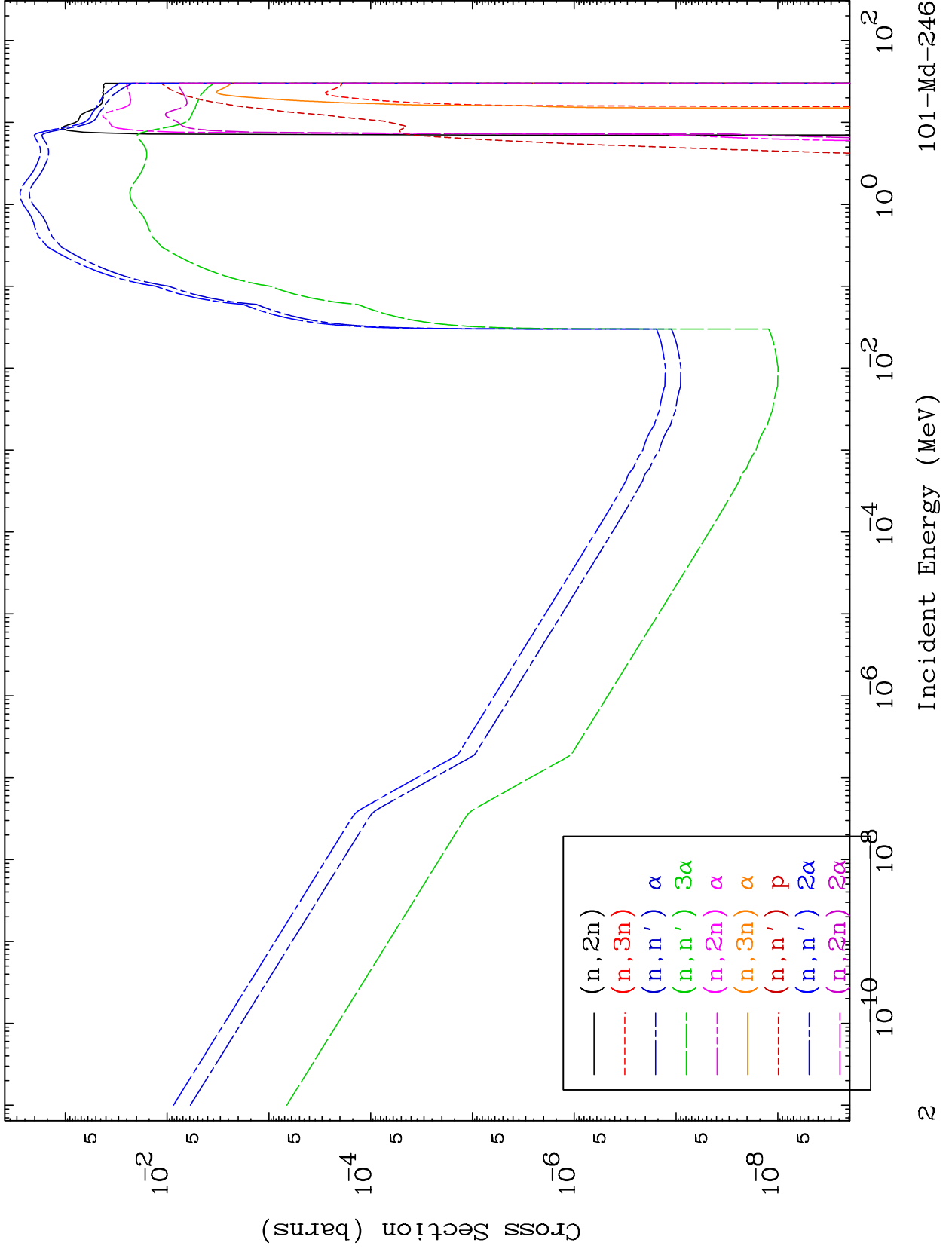
101-Md-246



MAT 9952

Neutron Production
293 Kelvin Cross Sections

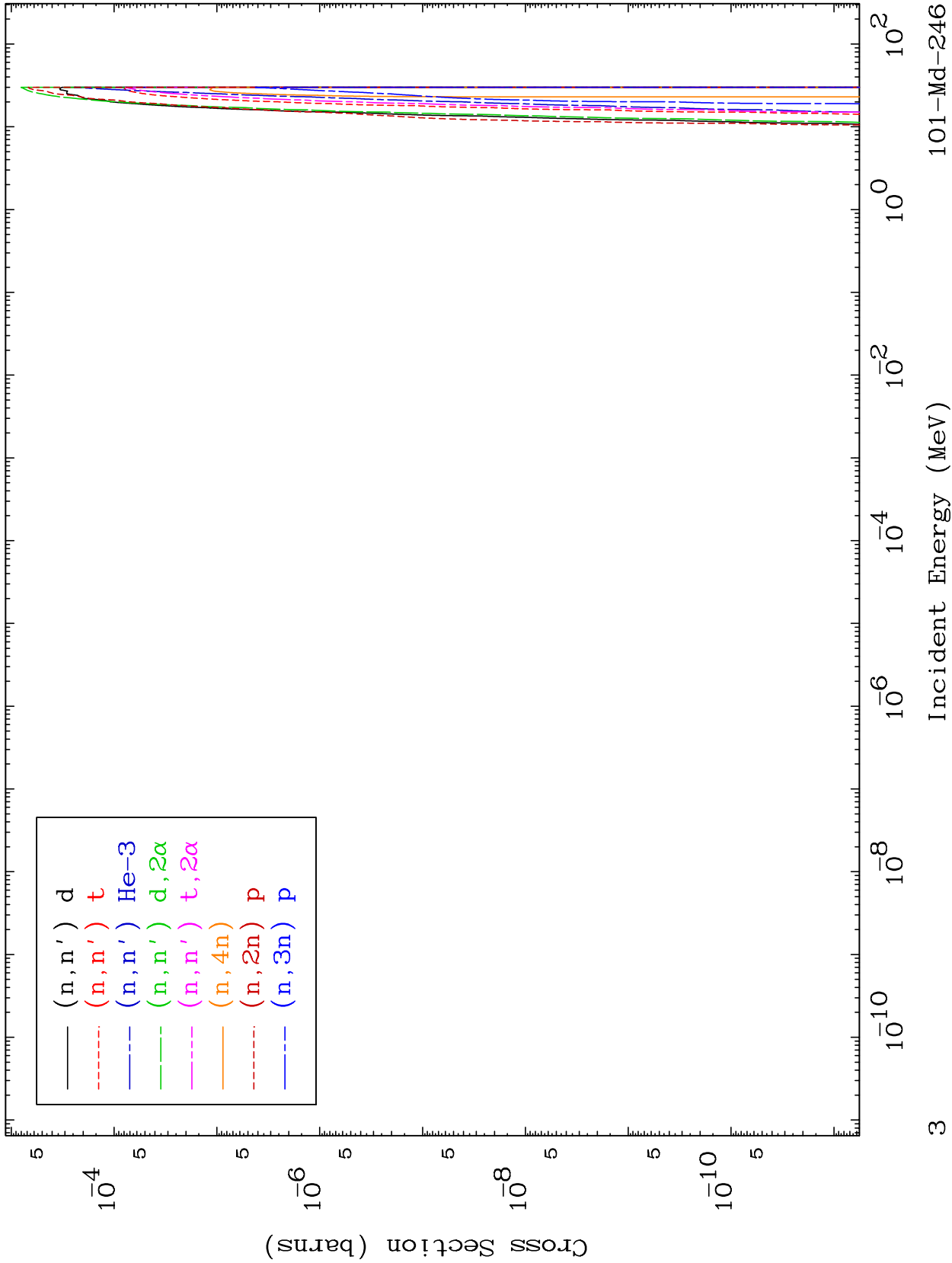
101-Md-246



MAT 9952

Neutron Production
293 Kelvin Cross Sections

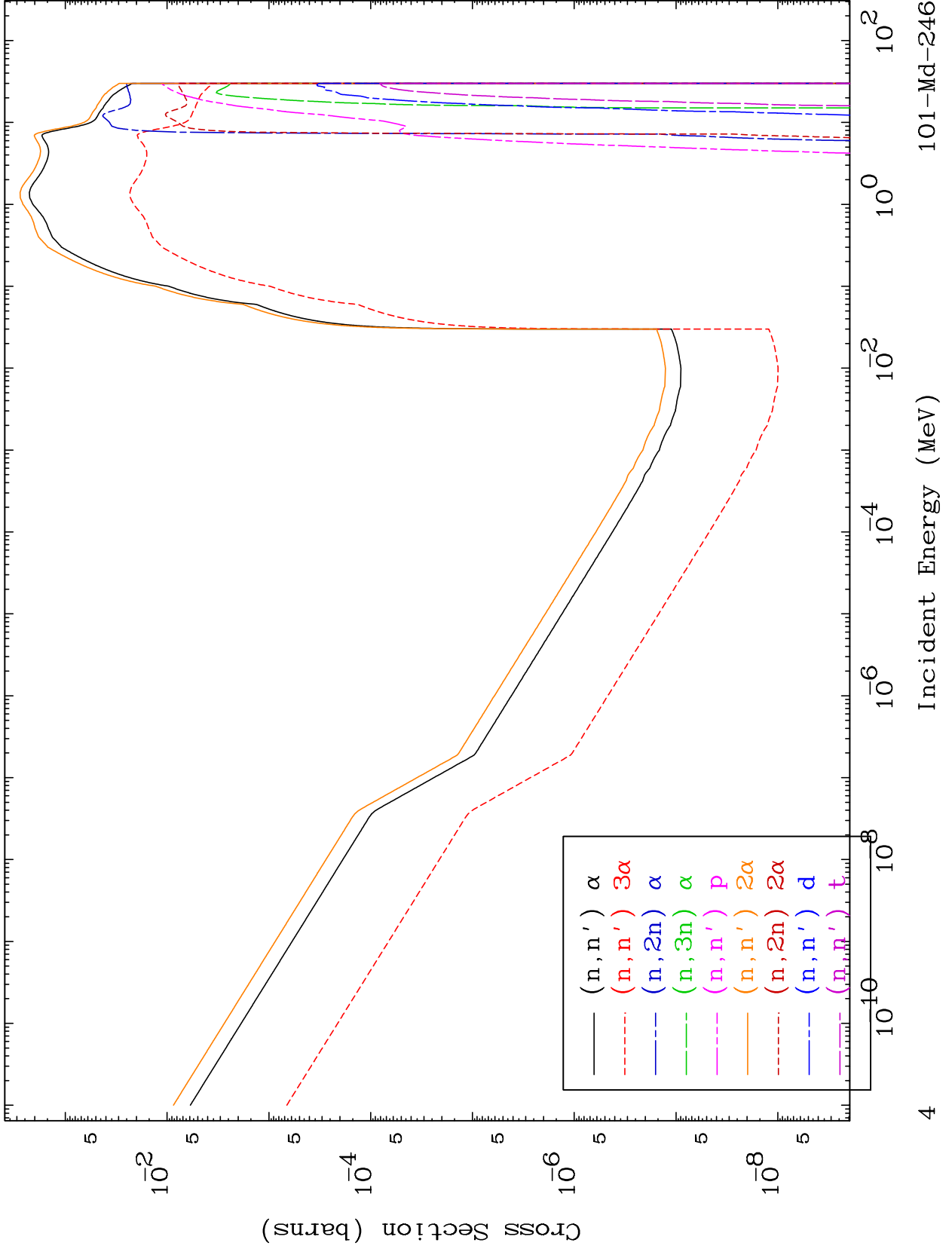
101-Md-246



MAT 9952

Charged Particle
293 Kelvin Cross Sections

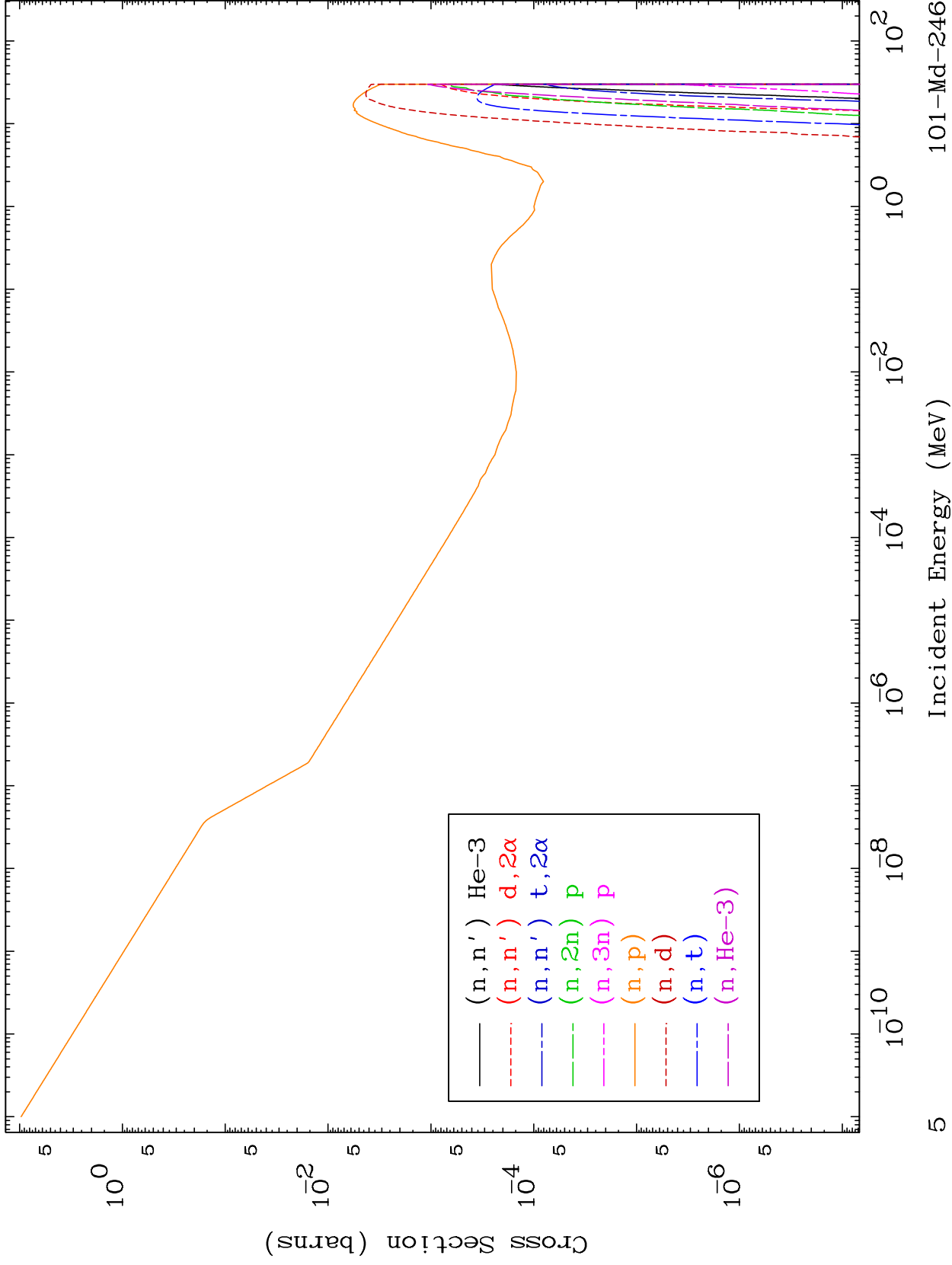
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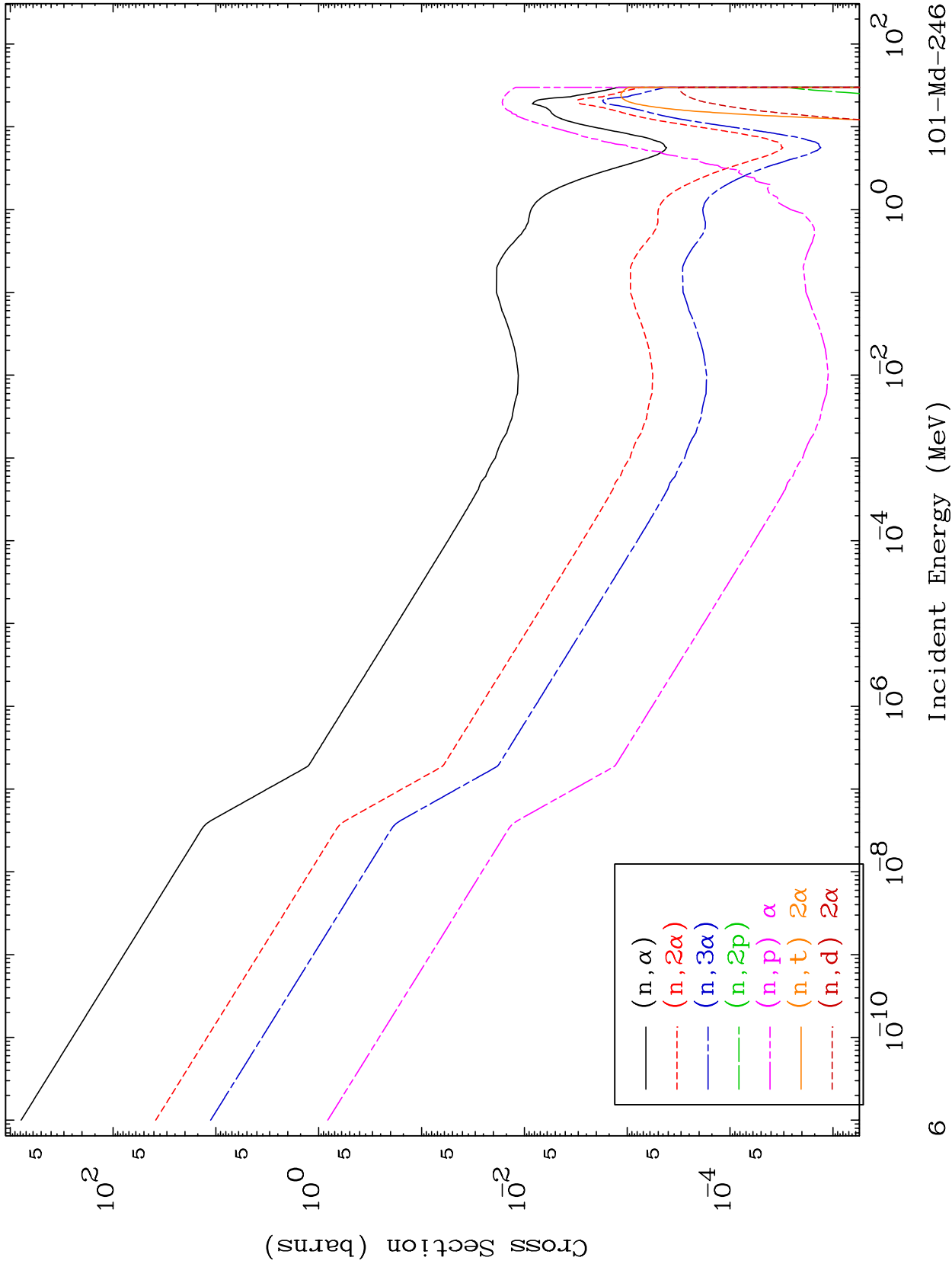


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Charged Particle
293 Kelvin Cross Sections

101-Md-246

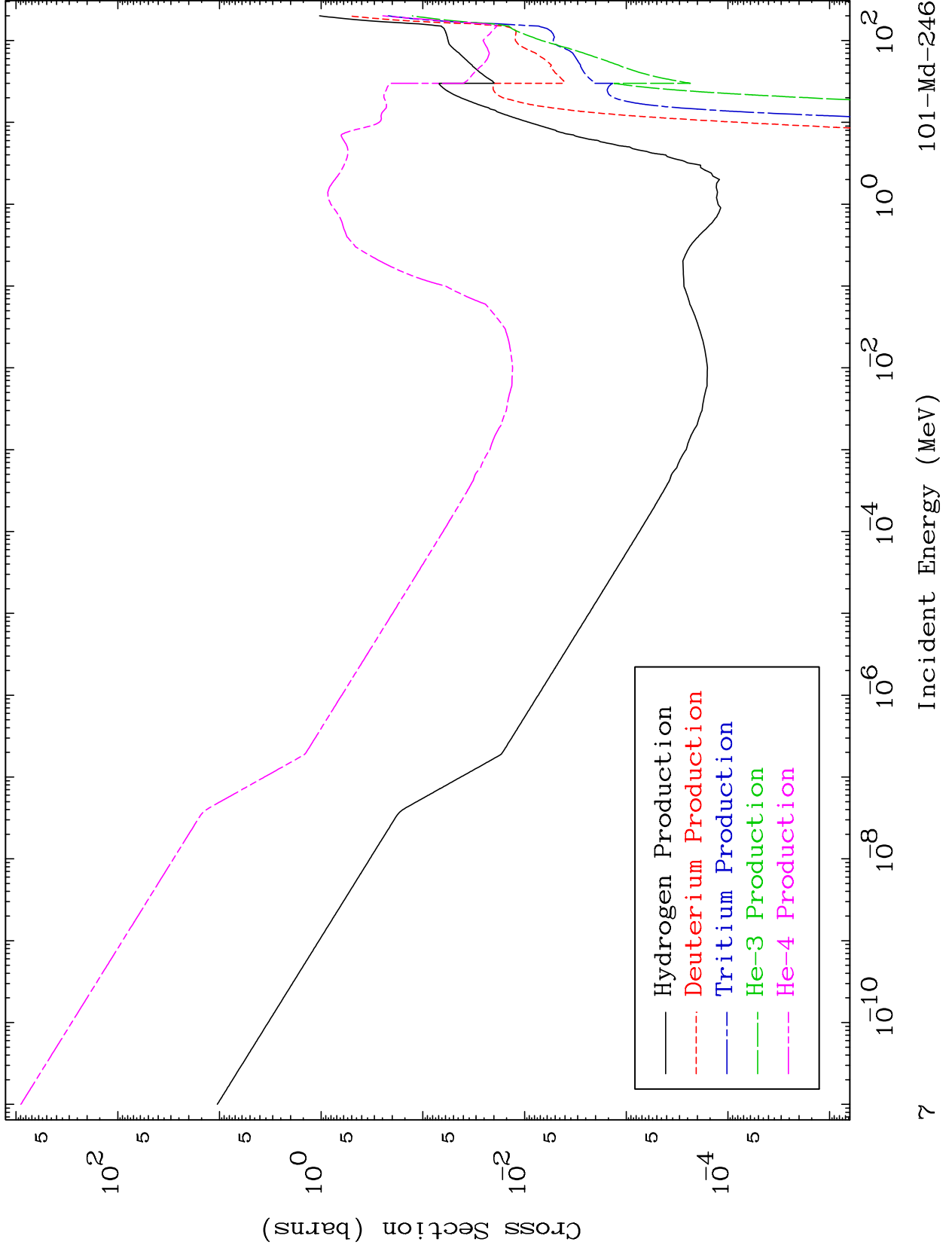




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Particle Production
293 Kelvin Cross Sections

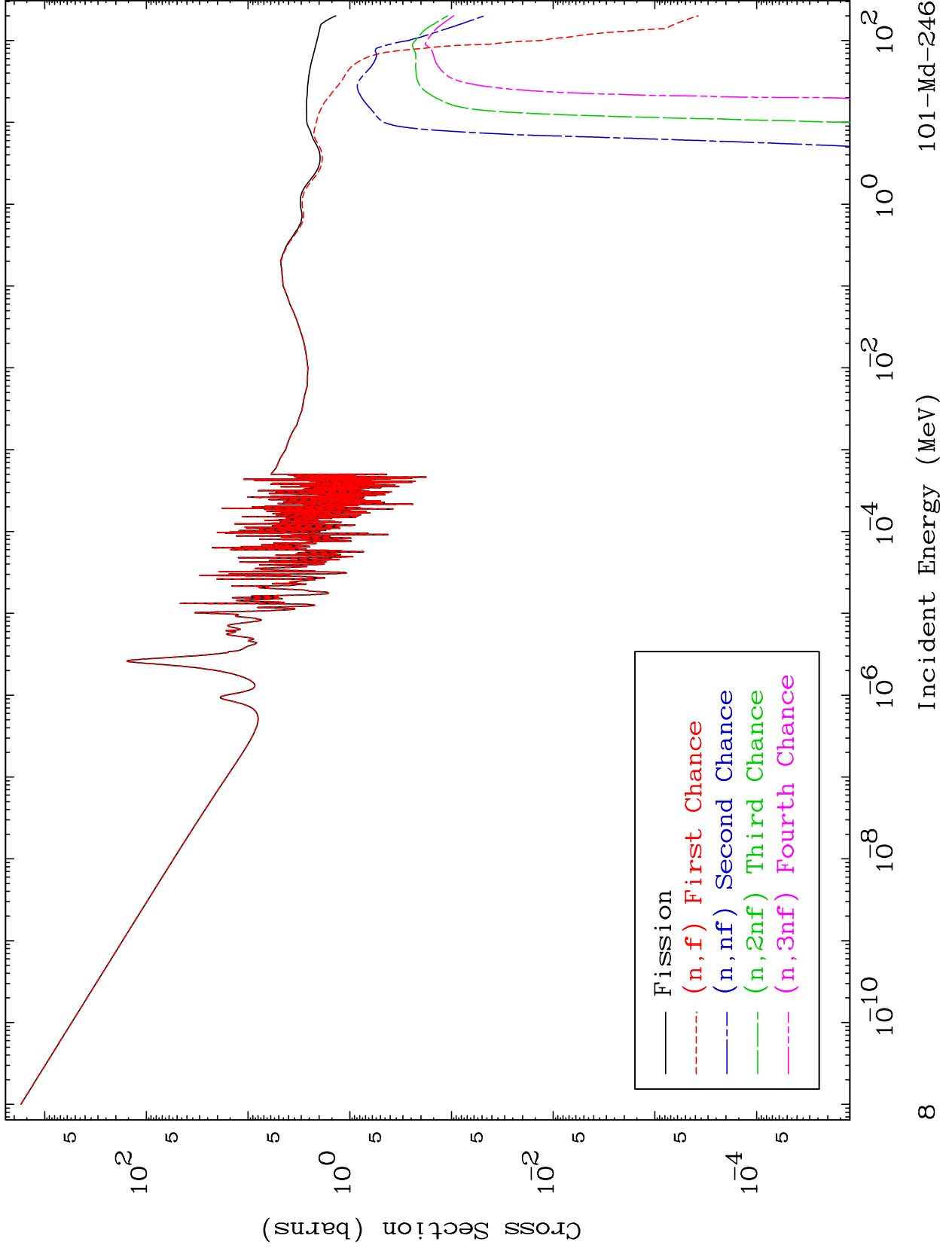
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Fission
293 Kelvin Cross Sections

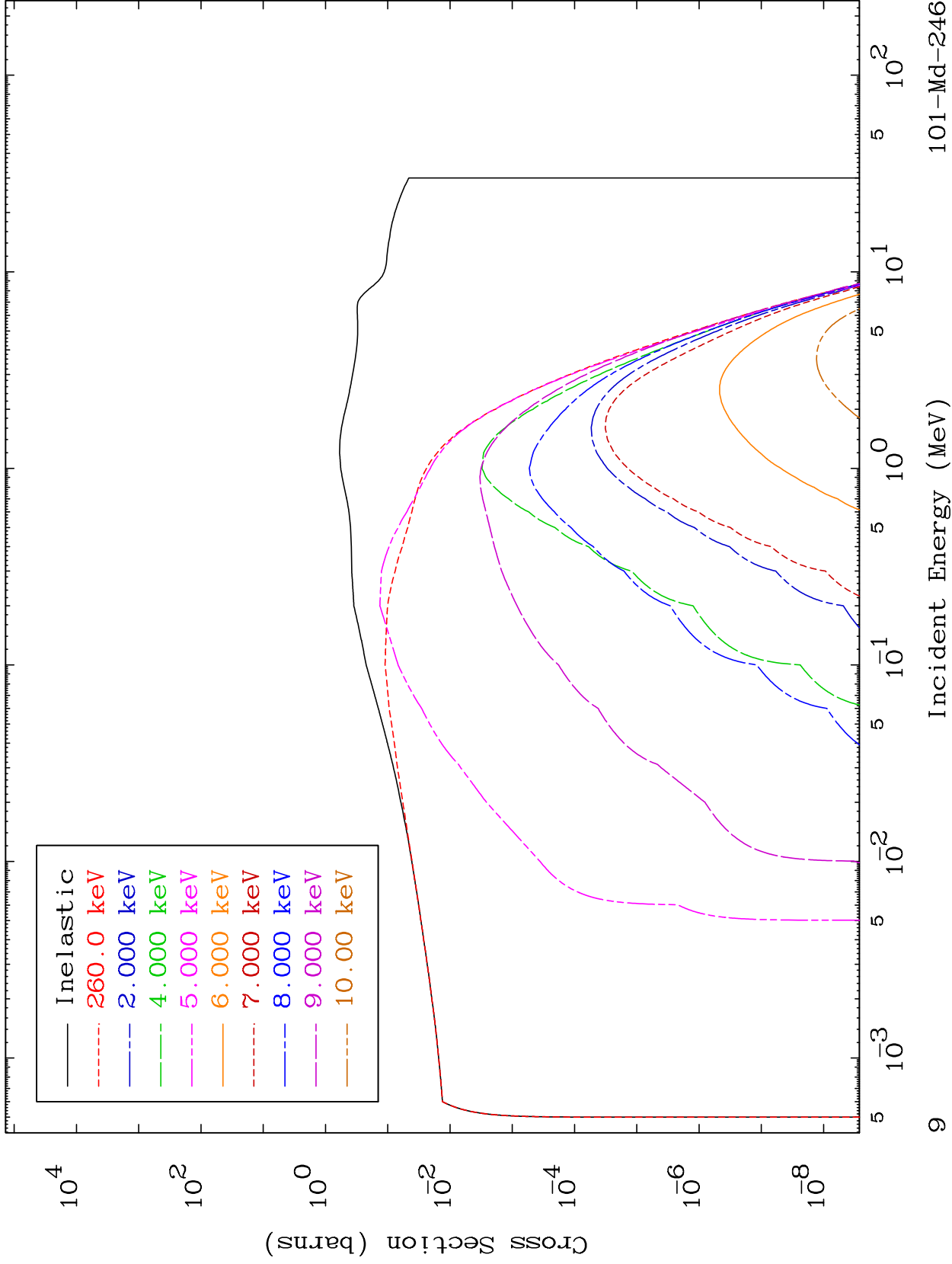
101-Md-246



MAT 9952

(n,n') Level
293 Kelvin Cross Sections

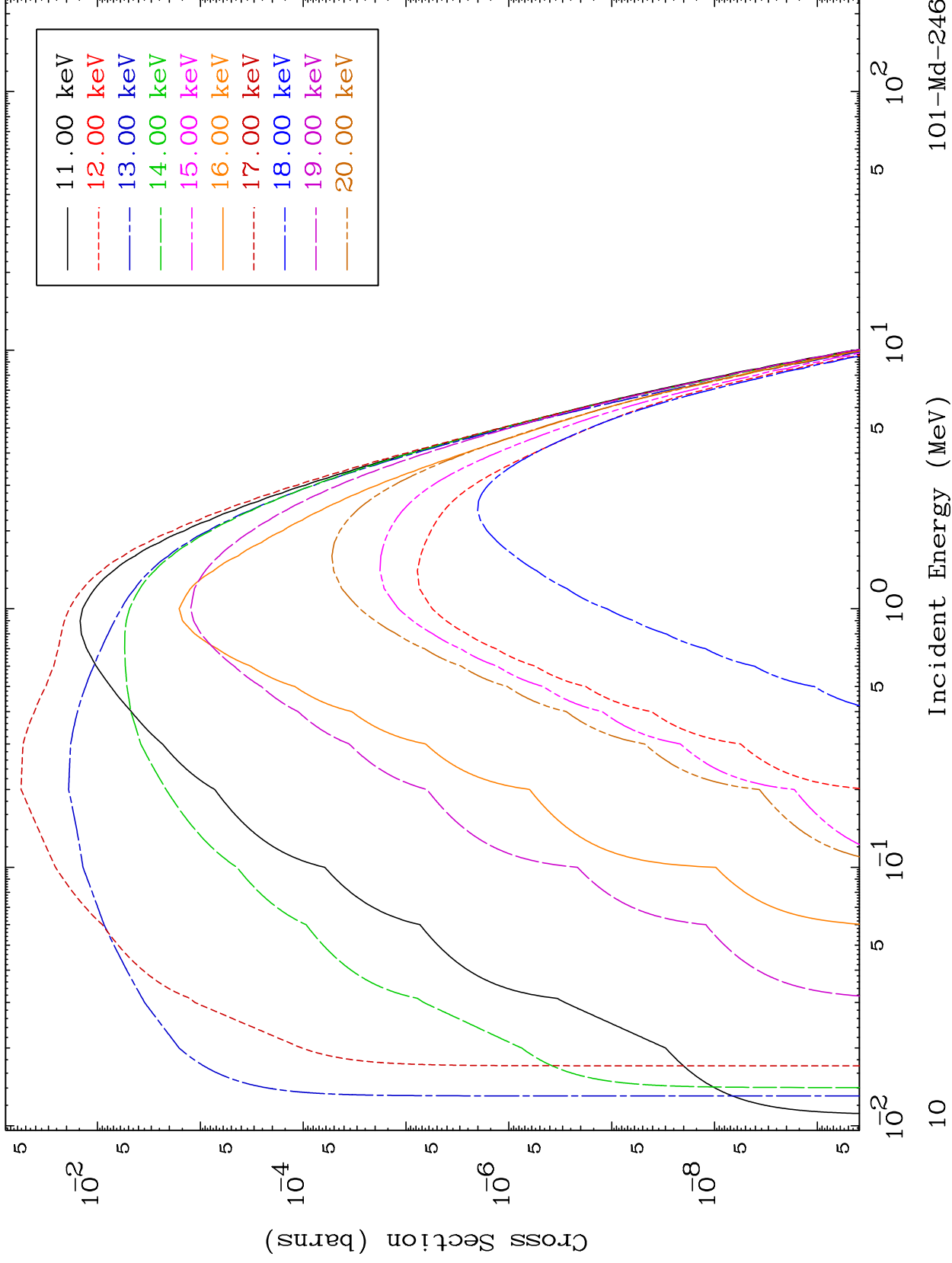
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MAT 9952

(n,n') Level
293 Kelvin Cross Sections

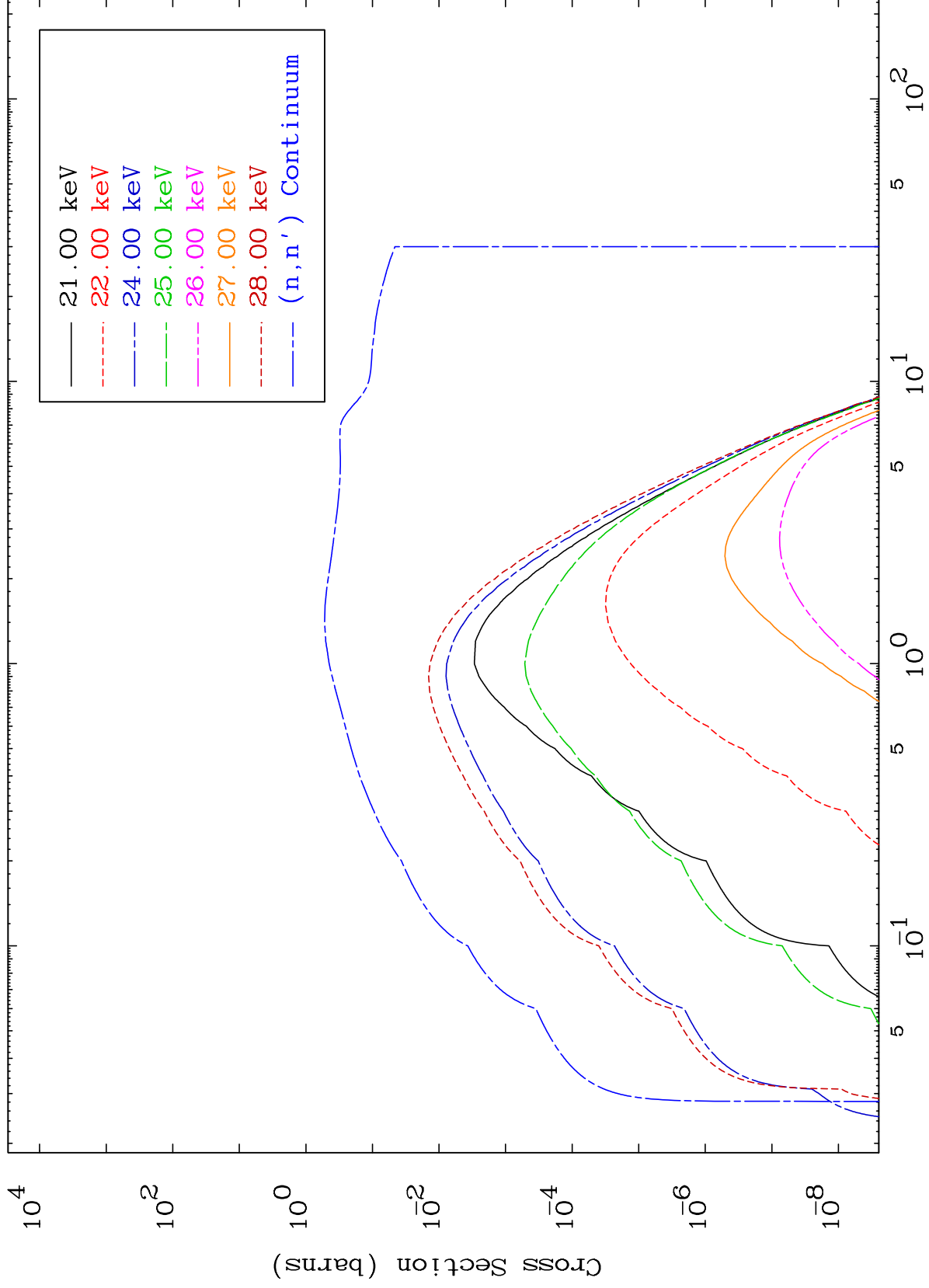
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(n,n') Level
293 Kelvin Cross Sections

101-Md-246



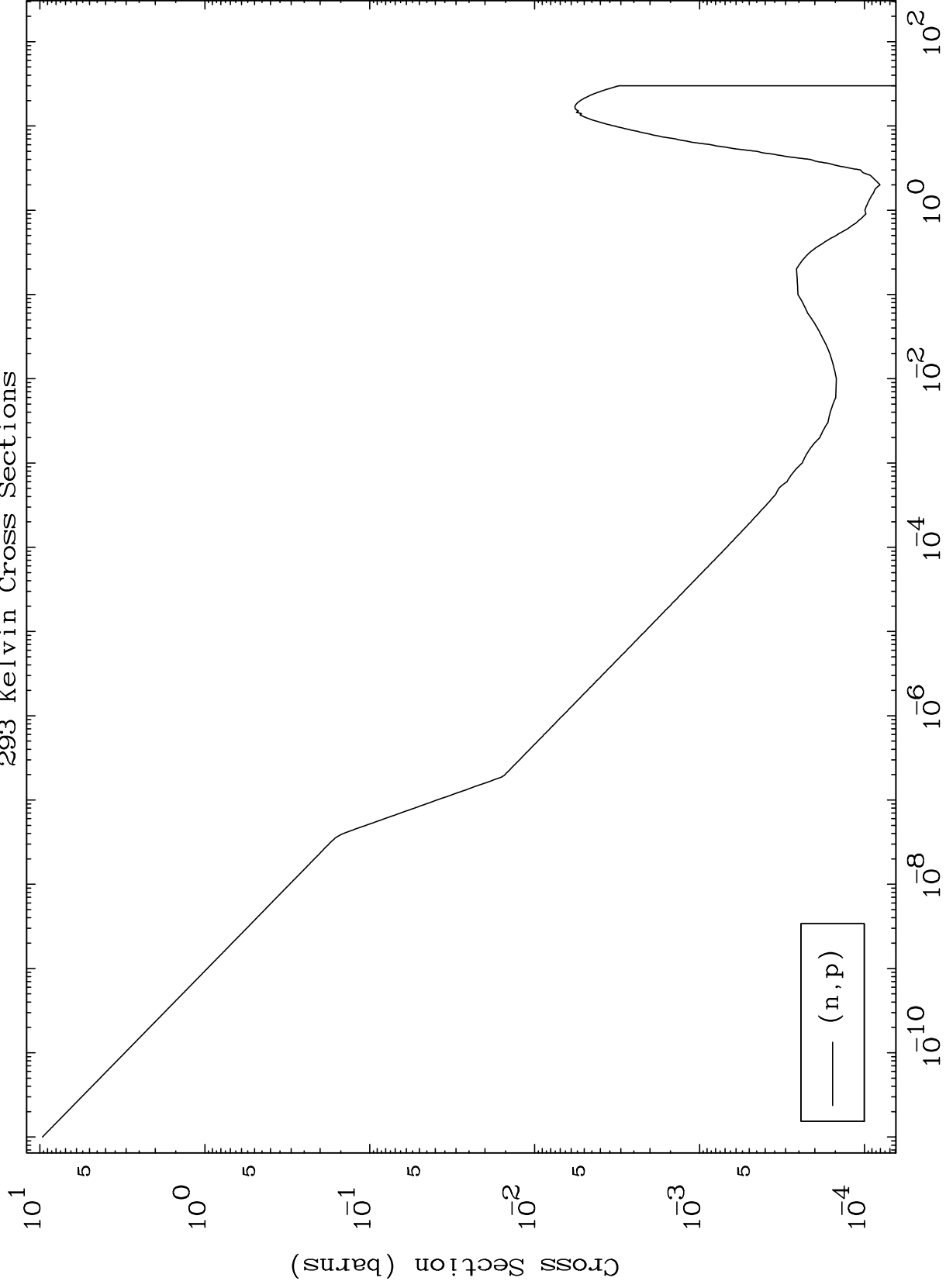
11

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(n,p) Levels
293 Kelvin Cross Sections

101-Md-246



12

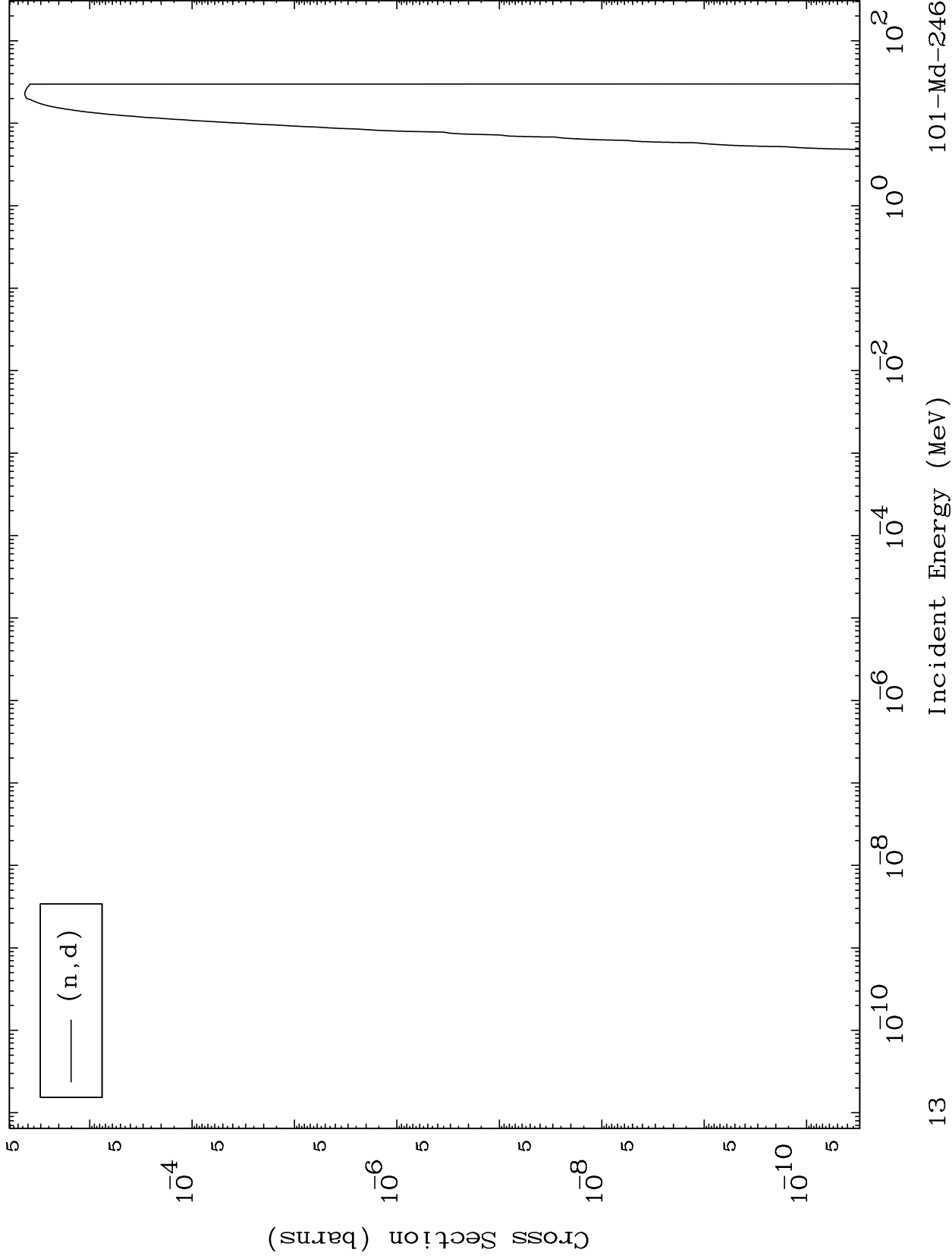
Incident Energy (MeV)

101-Md-246

MAT 9952

(n,d) Levels
293 Kelvin Cross Sections

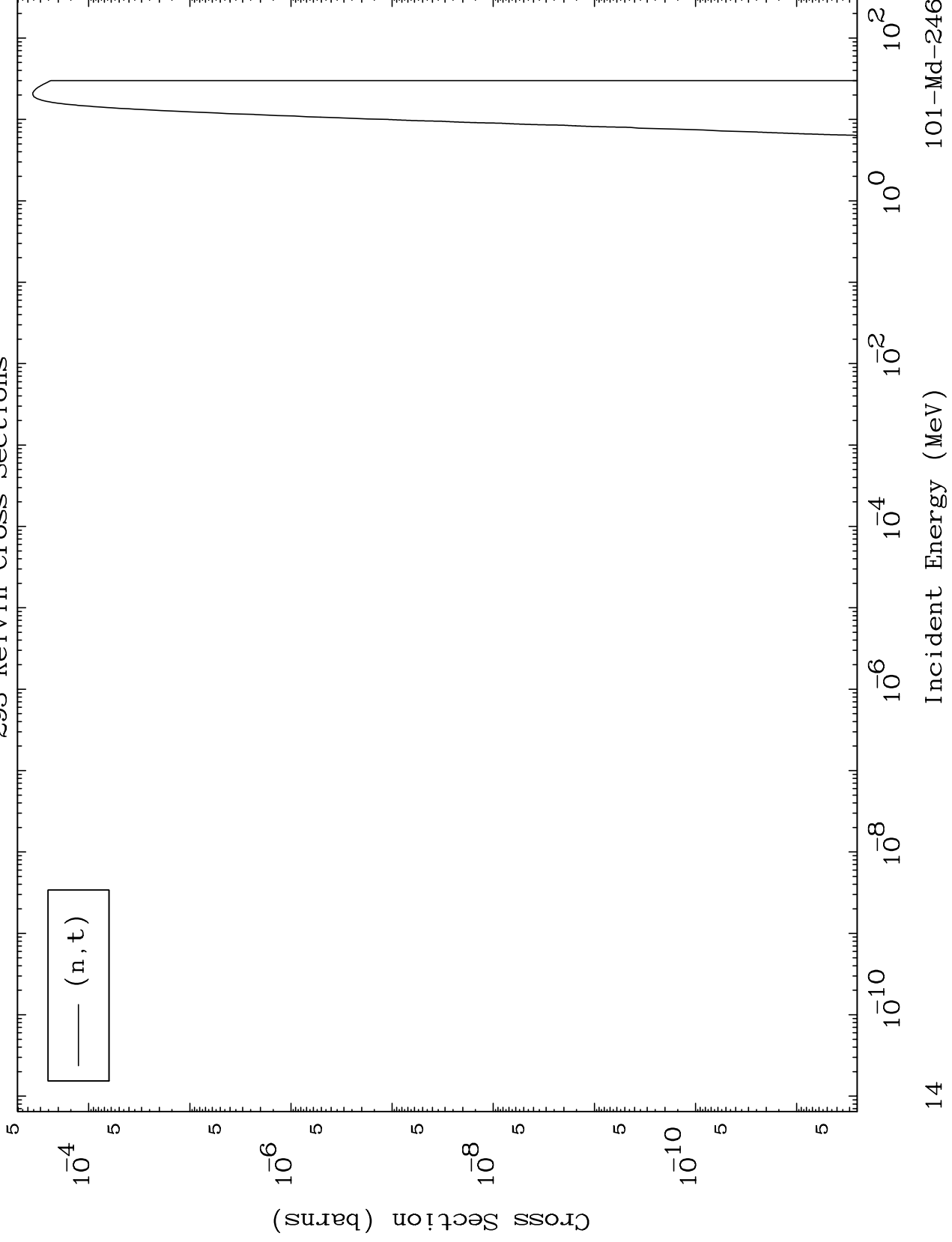
101-Md-246



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(n,t) Levels
293 Kelvin Cross Sections

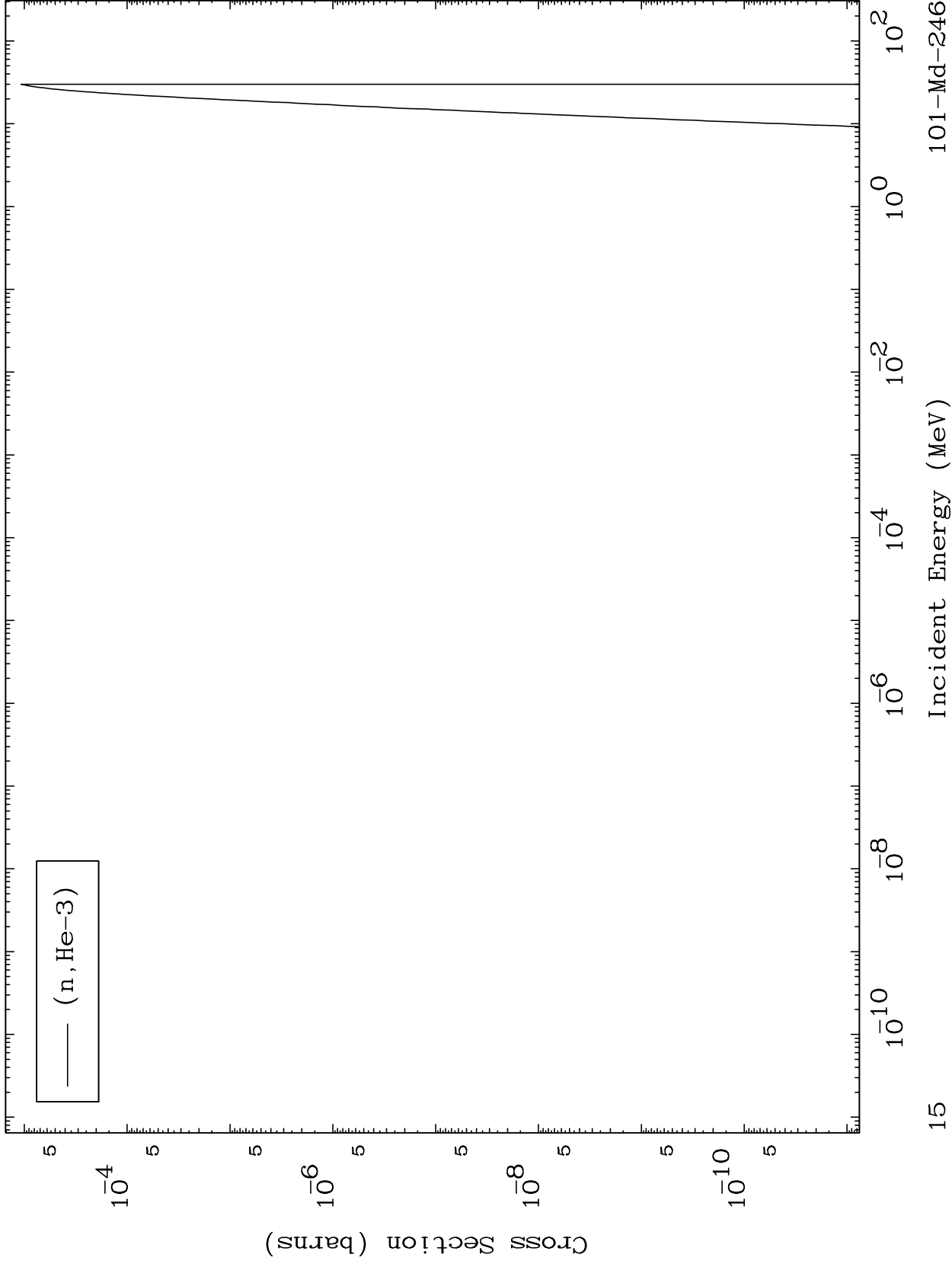
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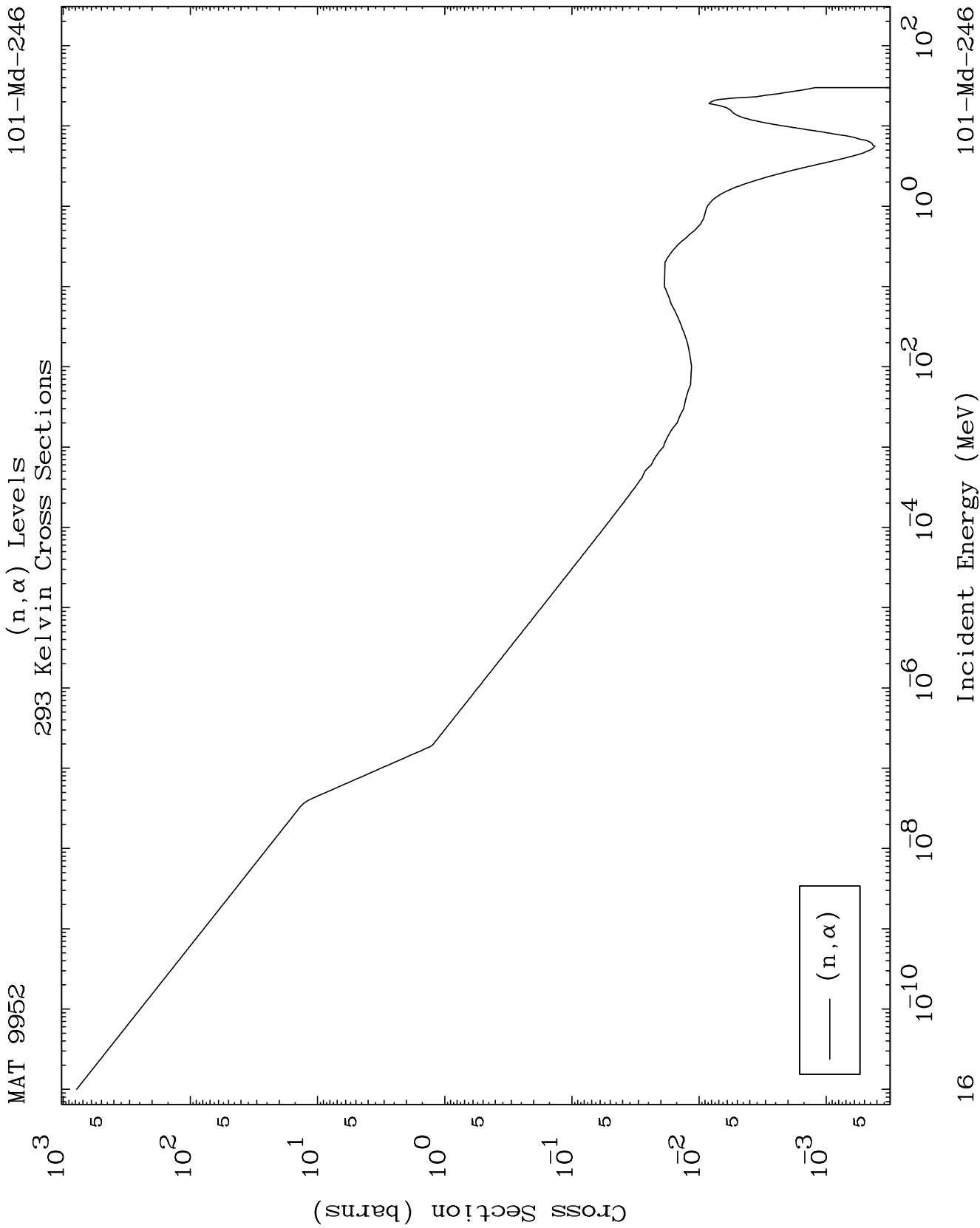


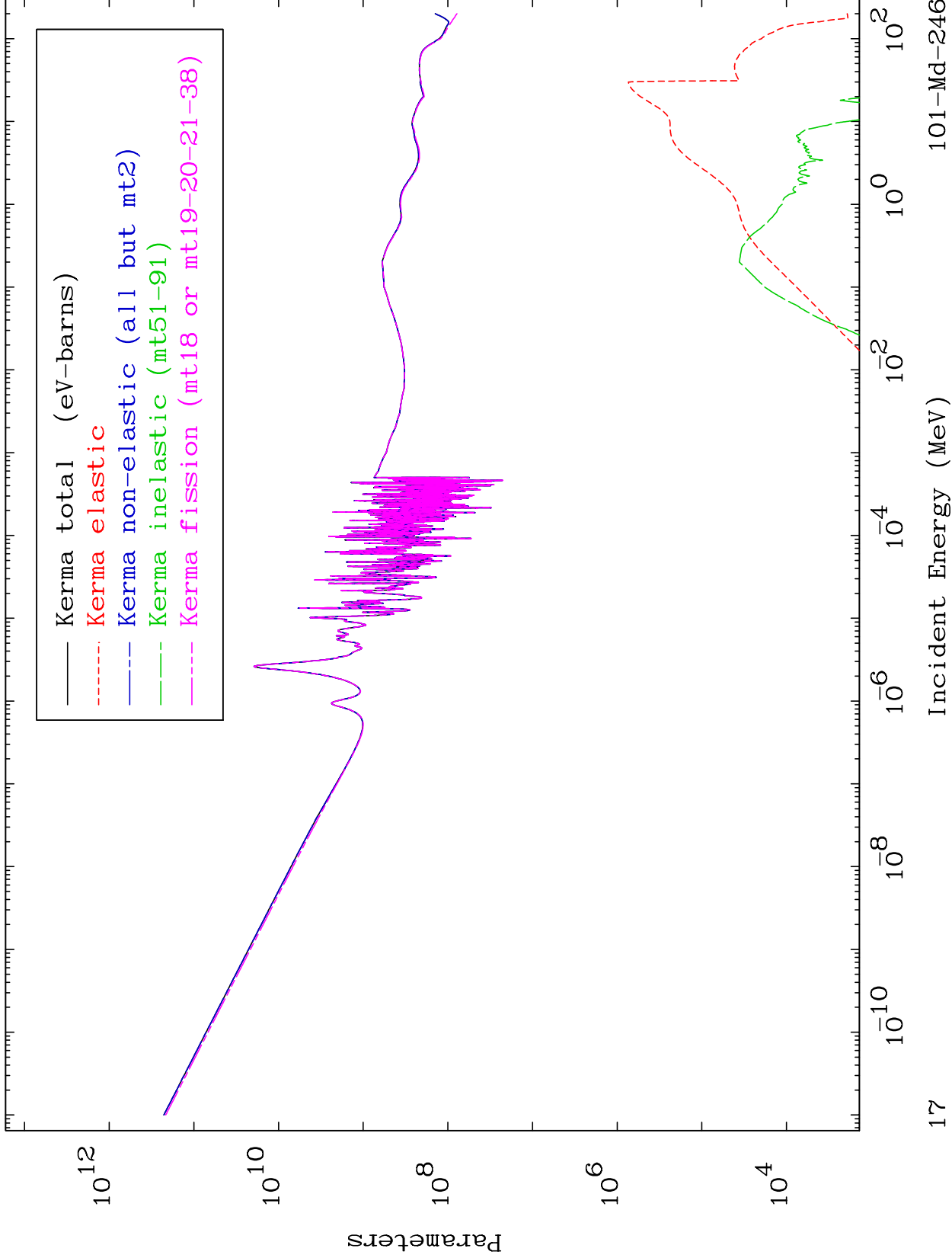
MAT 9952

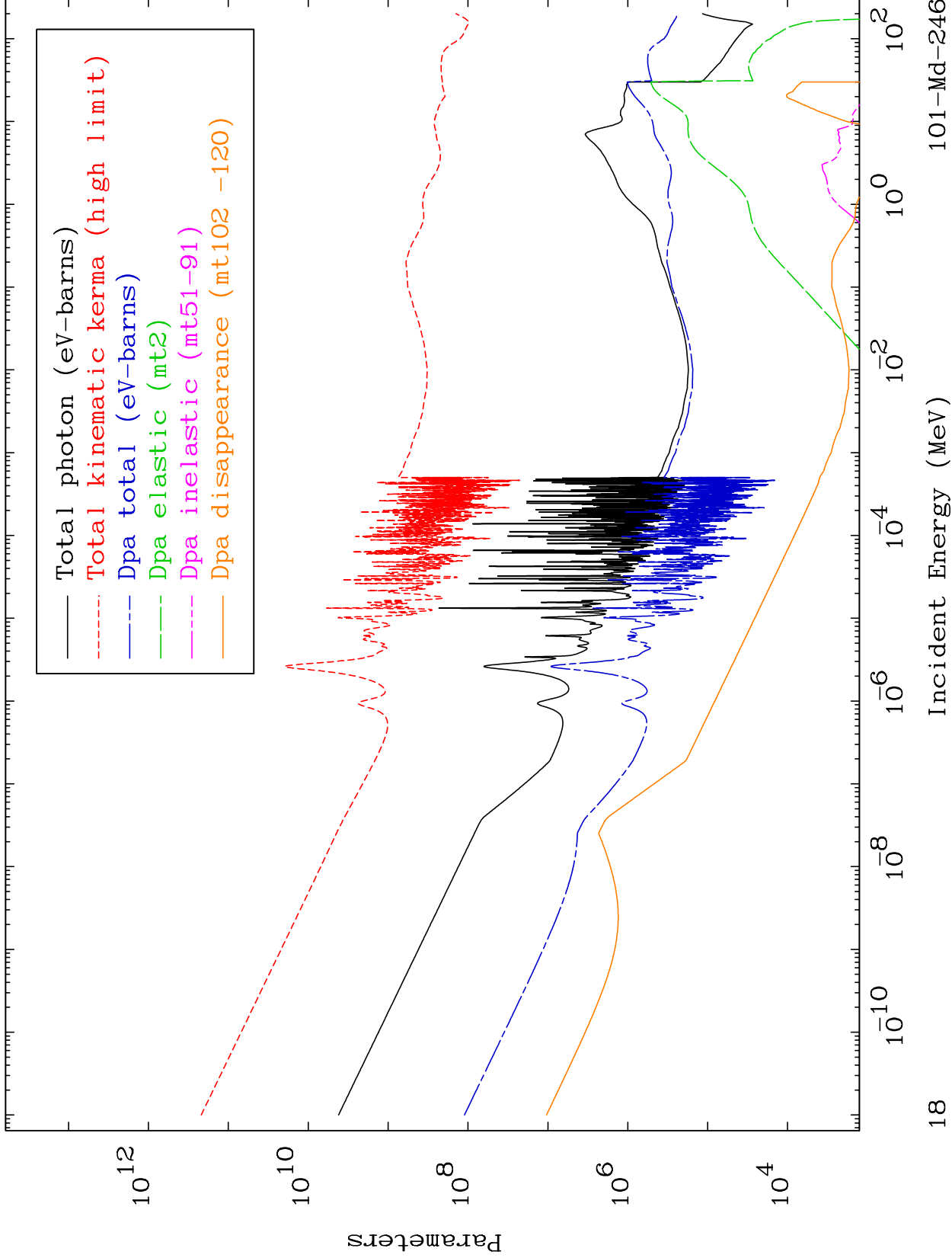
(n,He3) Levels
293 Kelvin Cross Sections

101-Md-246





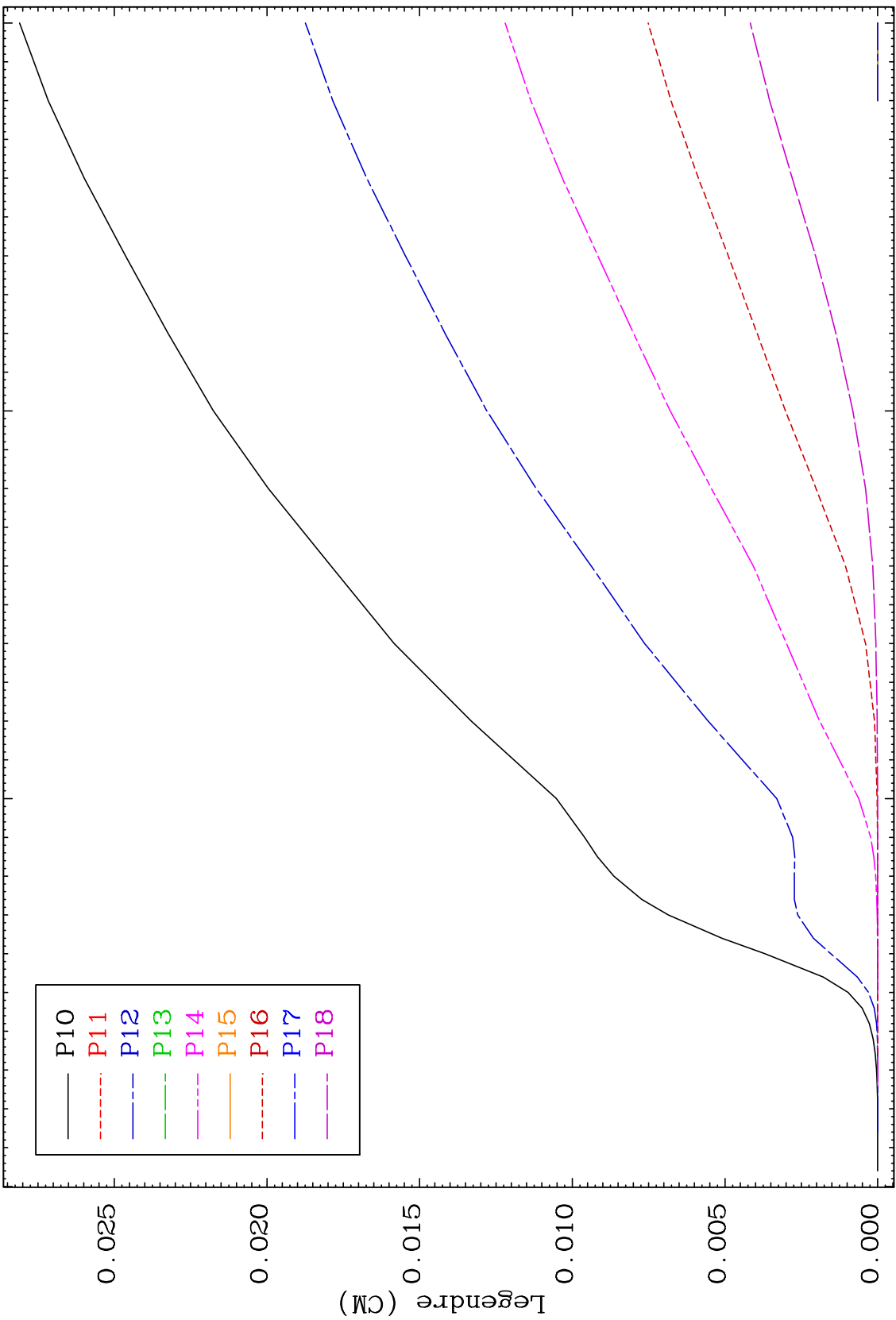




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Elastic Legendre Coefficients

101-Md-246



101-Md-246

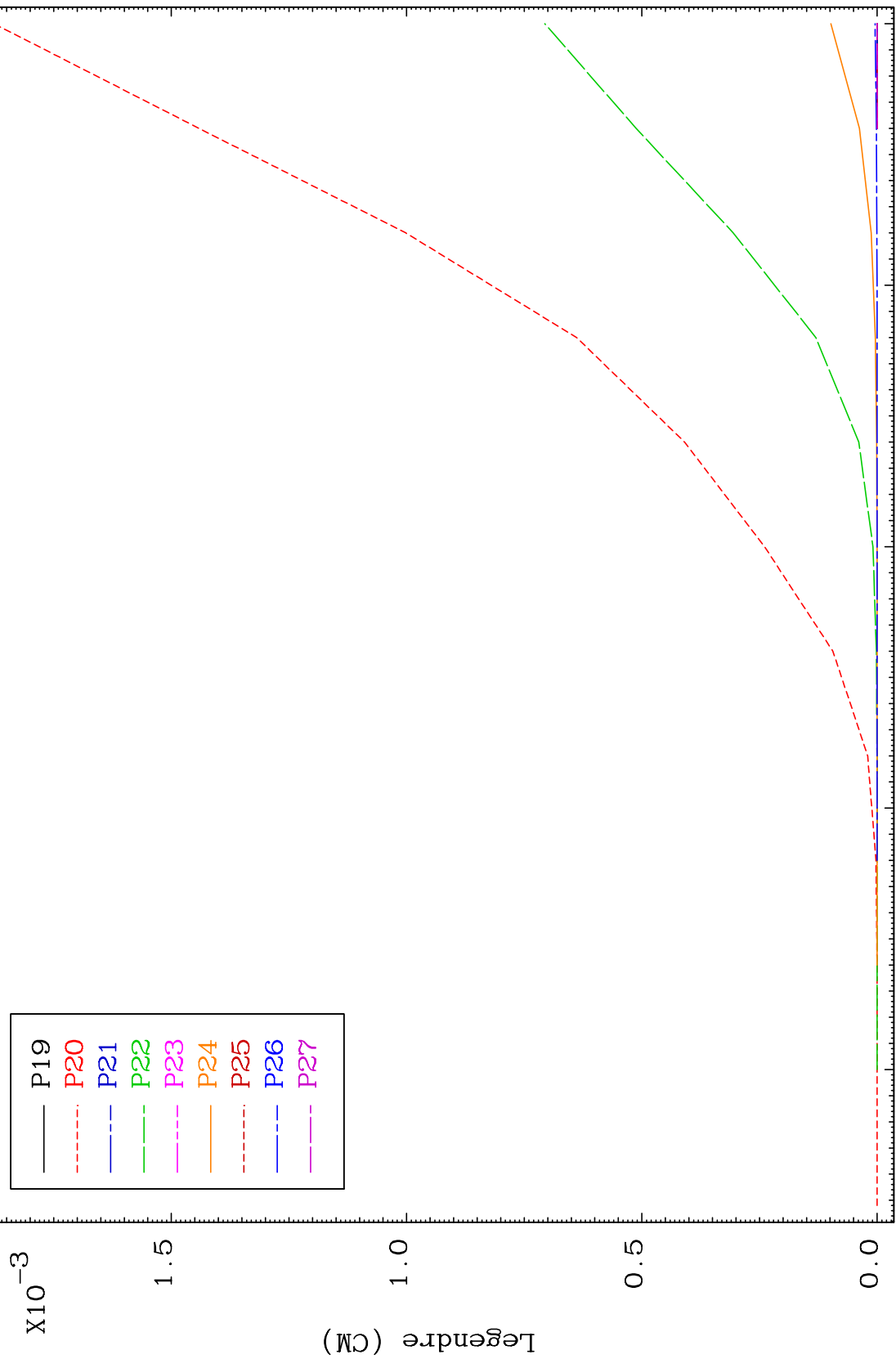
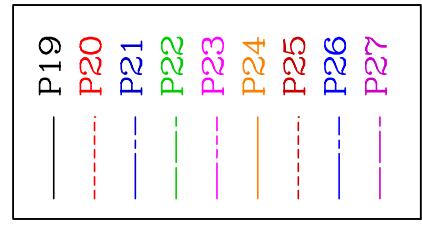
Incident Energy (MeV)

20

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Elastic Legendre Coefficients

101-Md-246



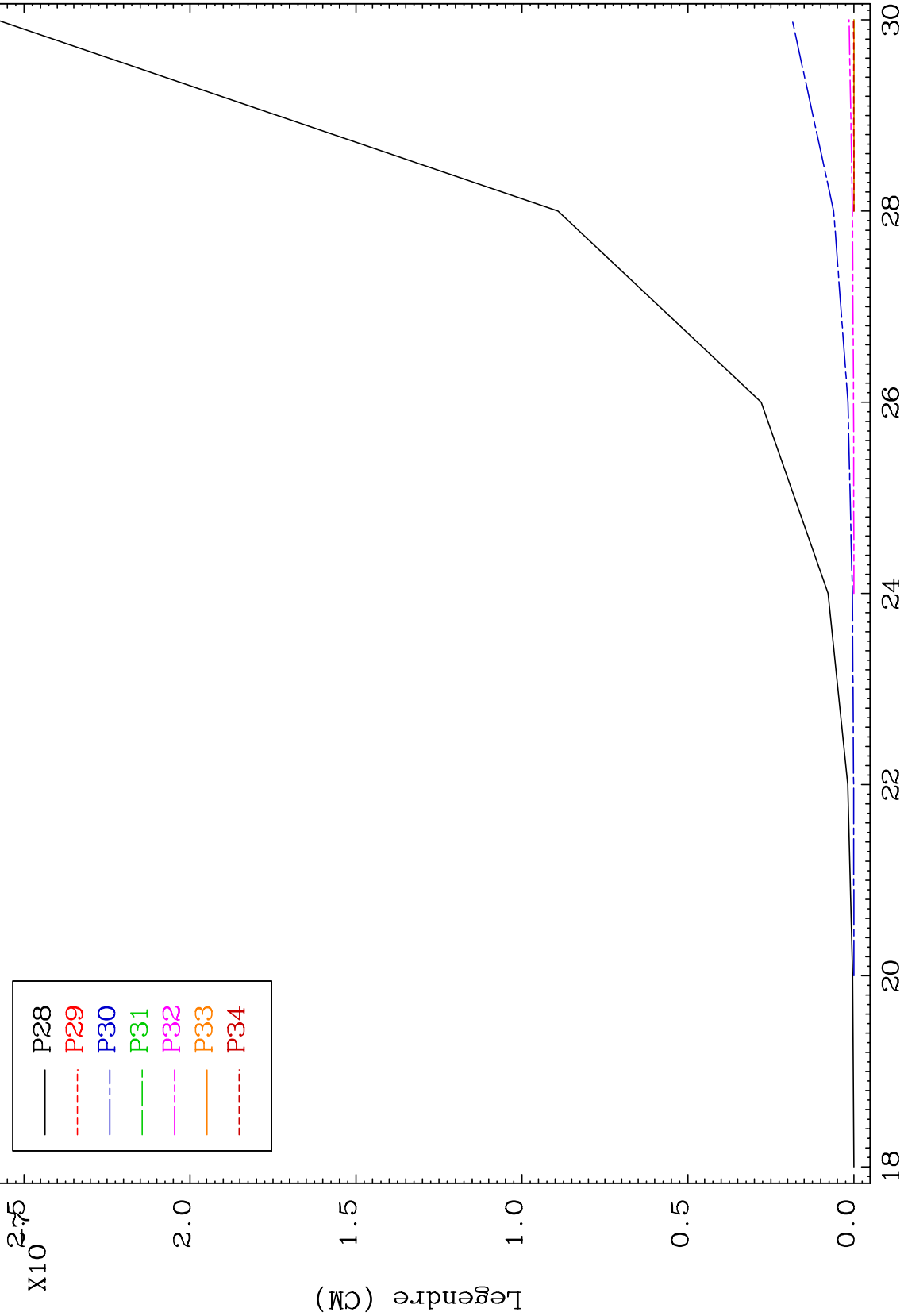
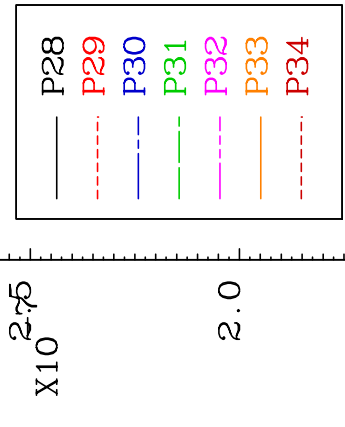
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Elastic Legendre Coefficients

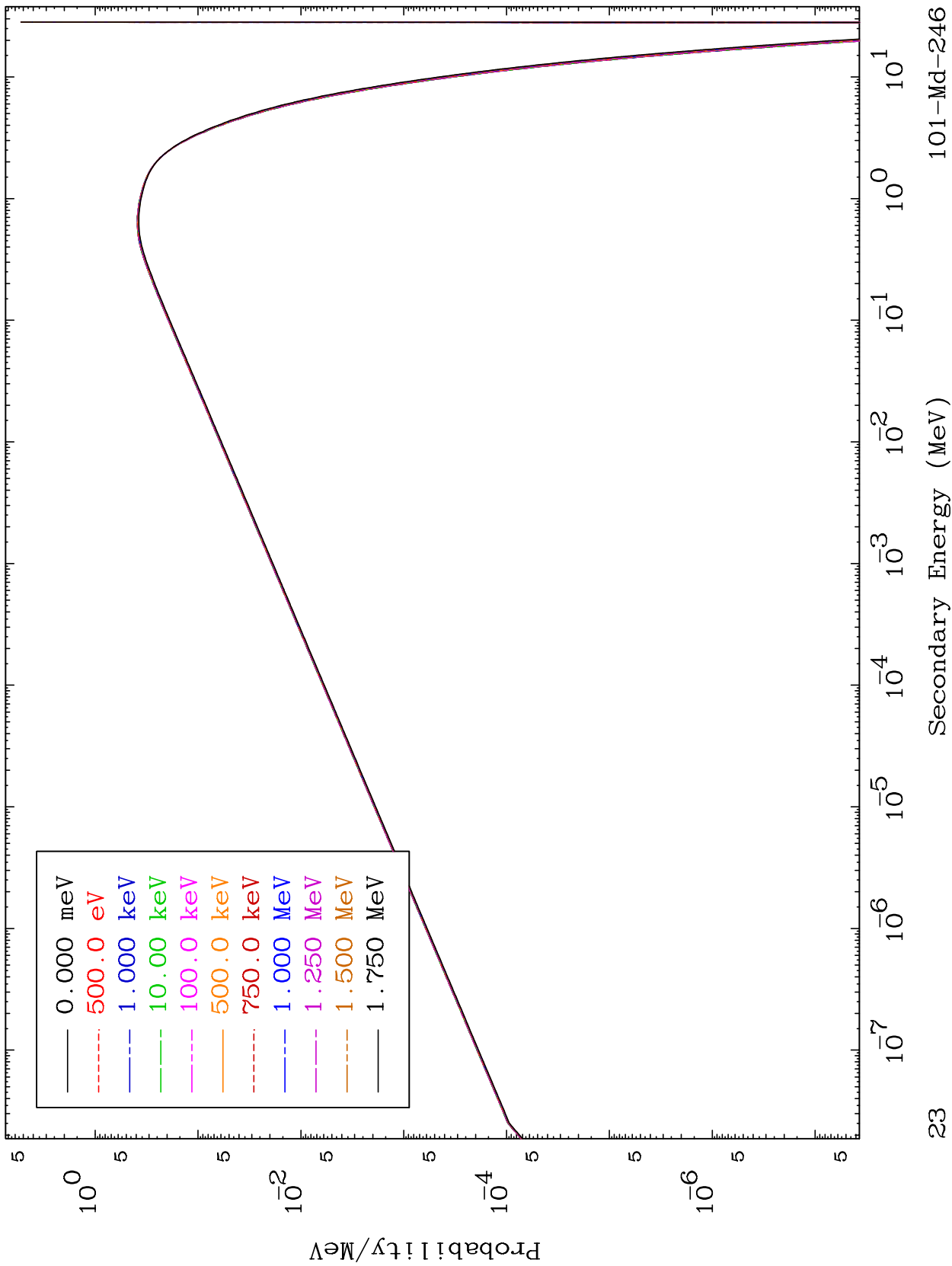
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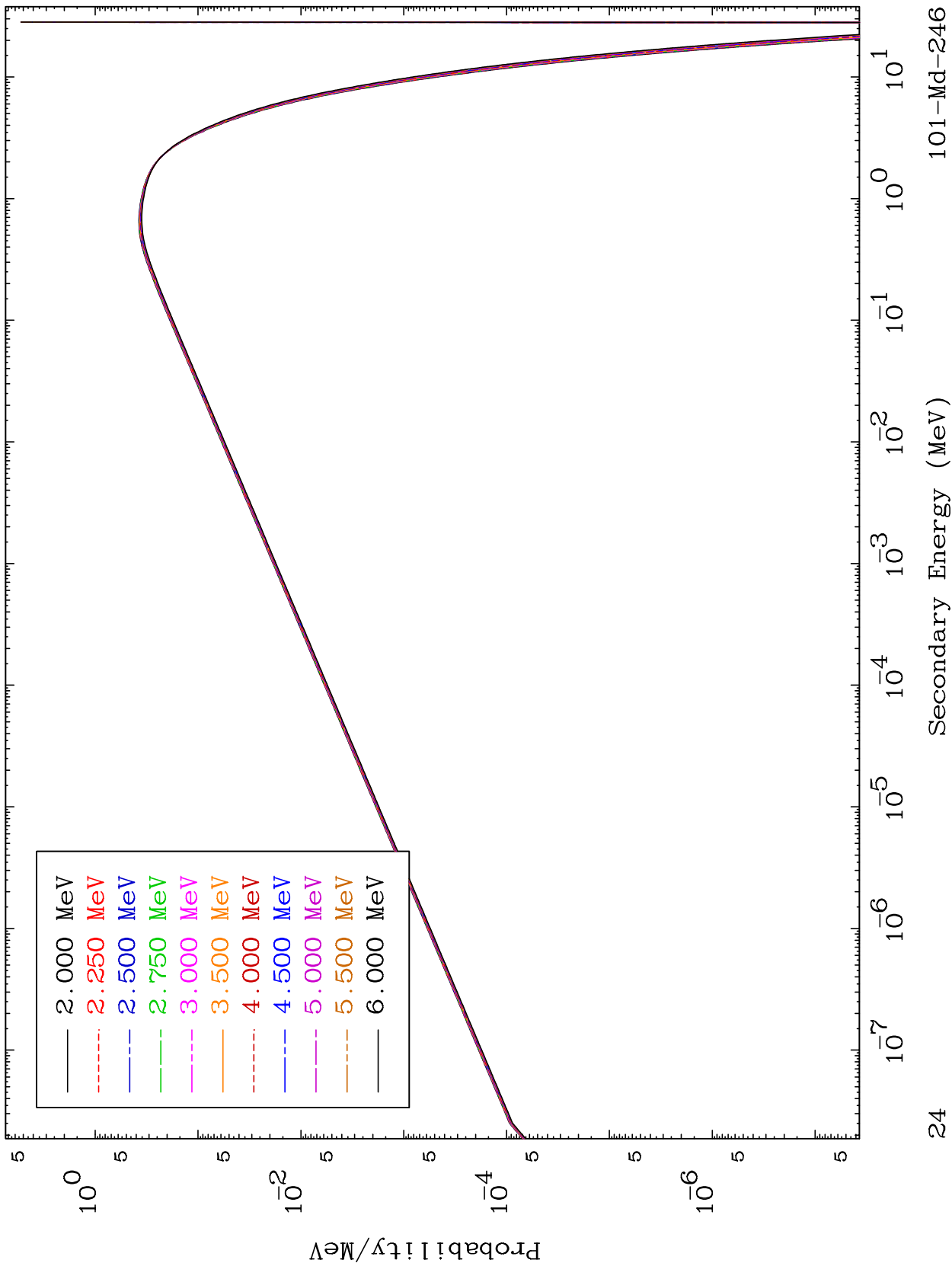
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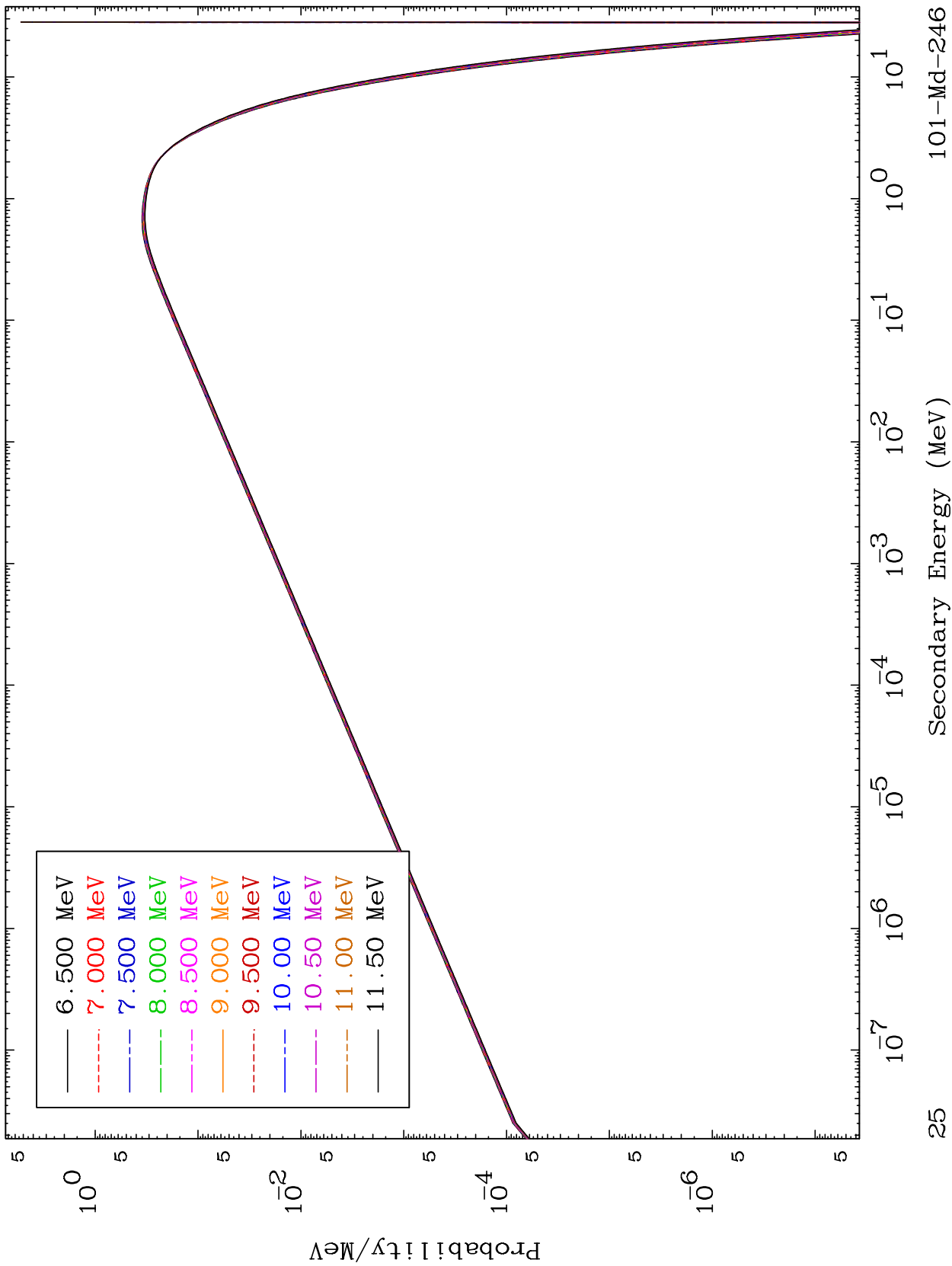
Incident Energy (MeV)

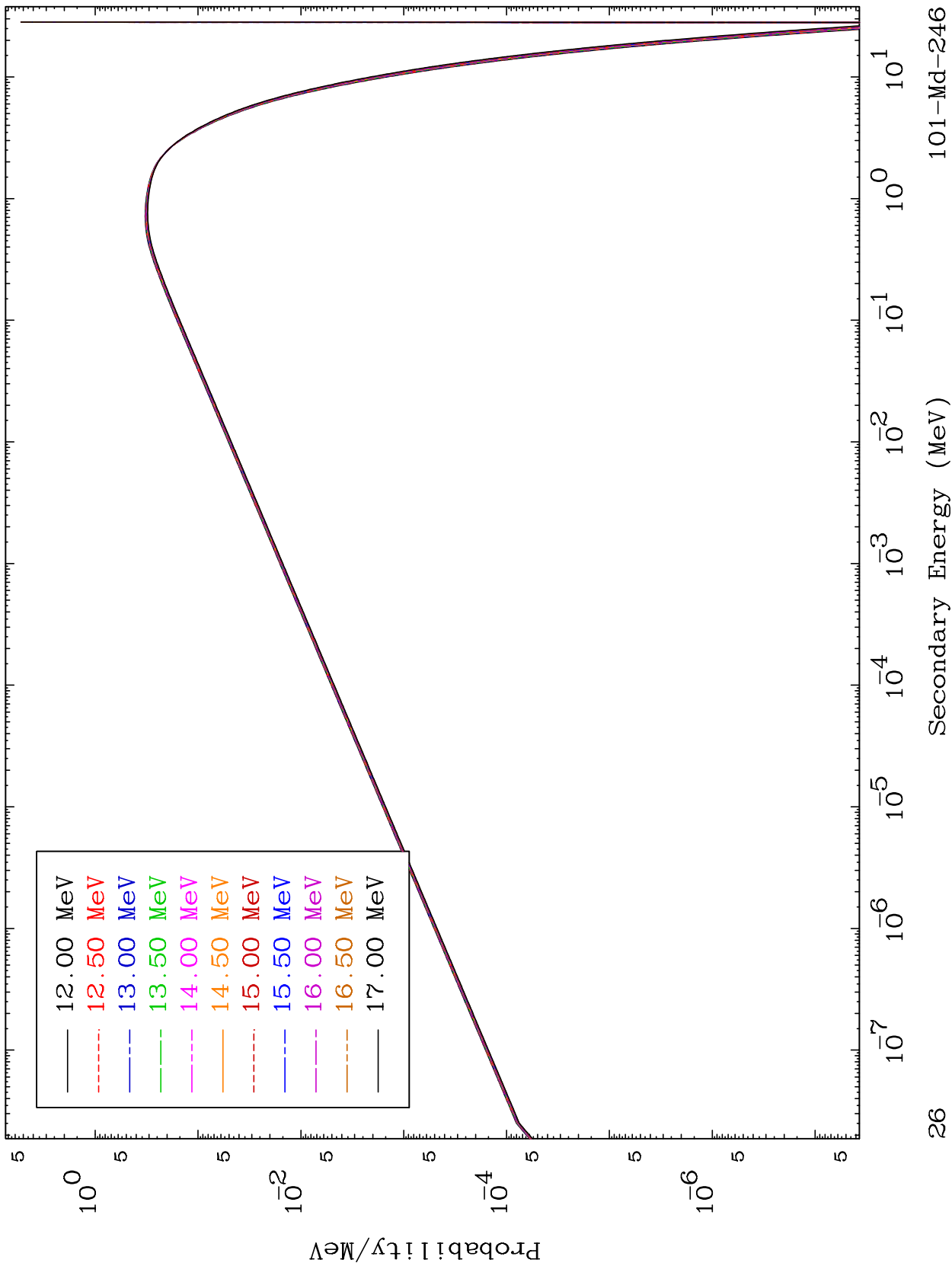
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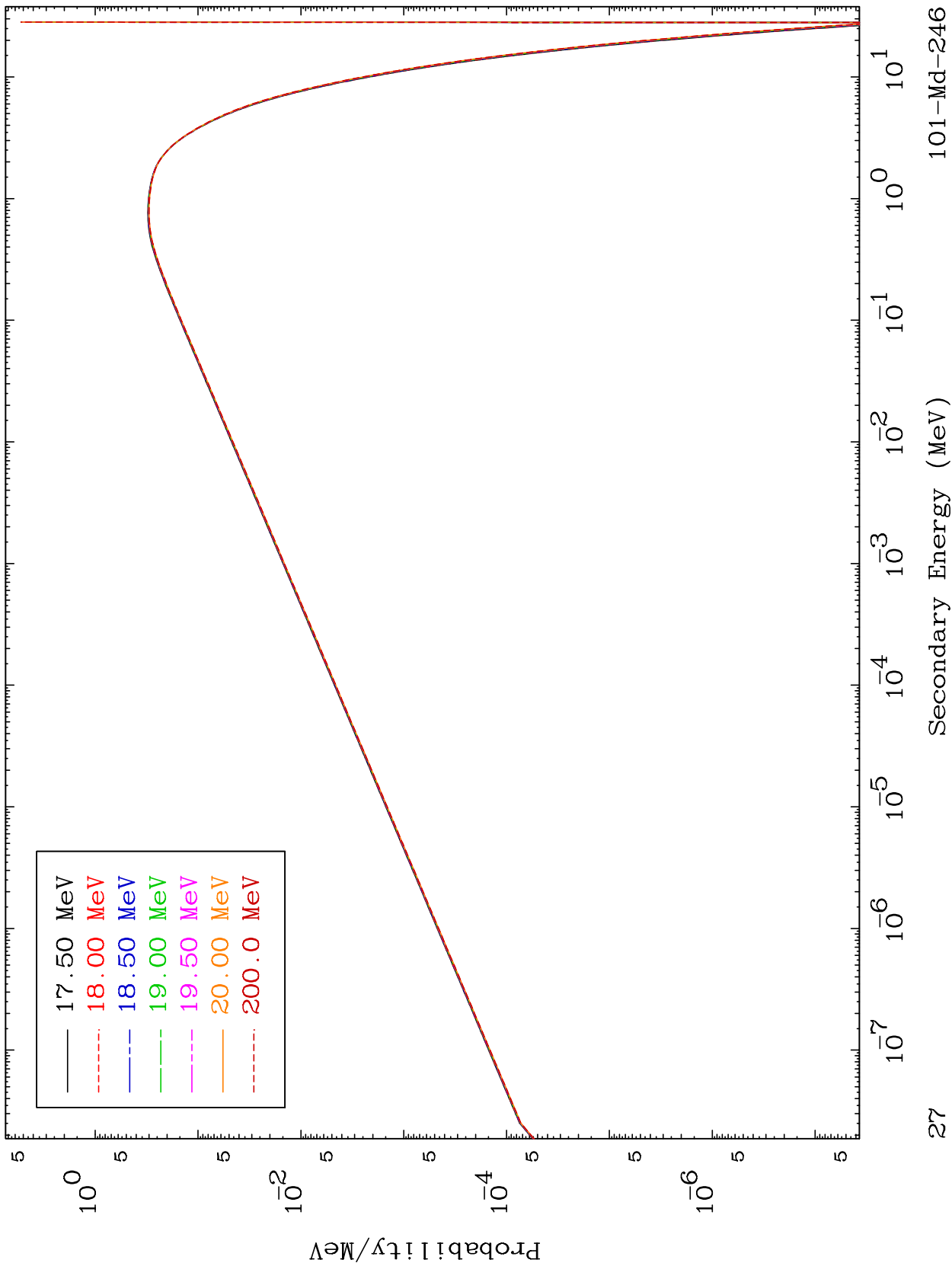


Fission
Energy Distribution





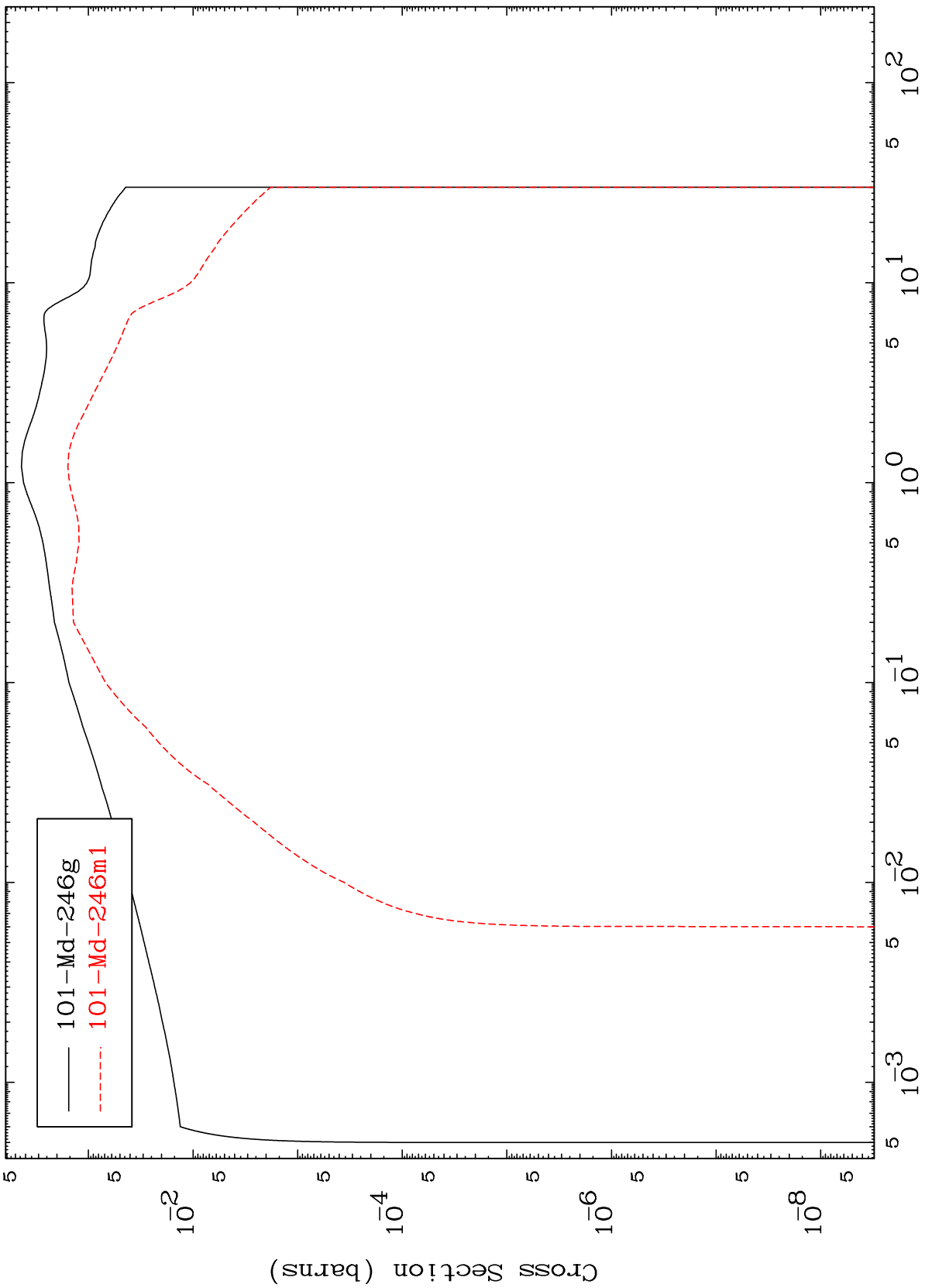




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101-Md-246

Inelastic
Radionuclide Production Cross Section



101-Md-246

Incident Energy (MeV)