

Program EVALPLOT
(Version 2015-2)

by

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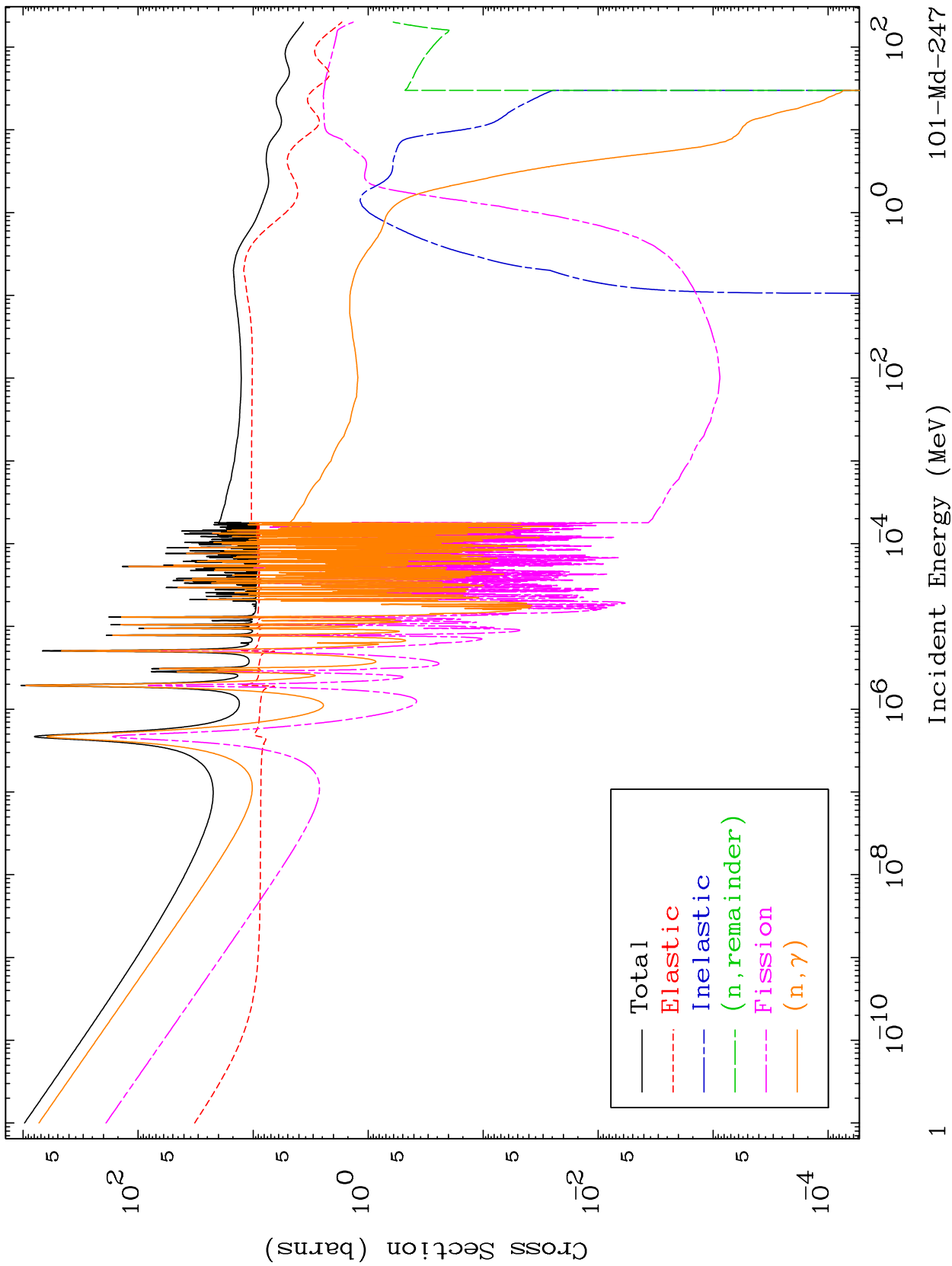
Press Mouse Button to Start

MAT 9948

Major

293 Kelvin Cross Sections

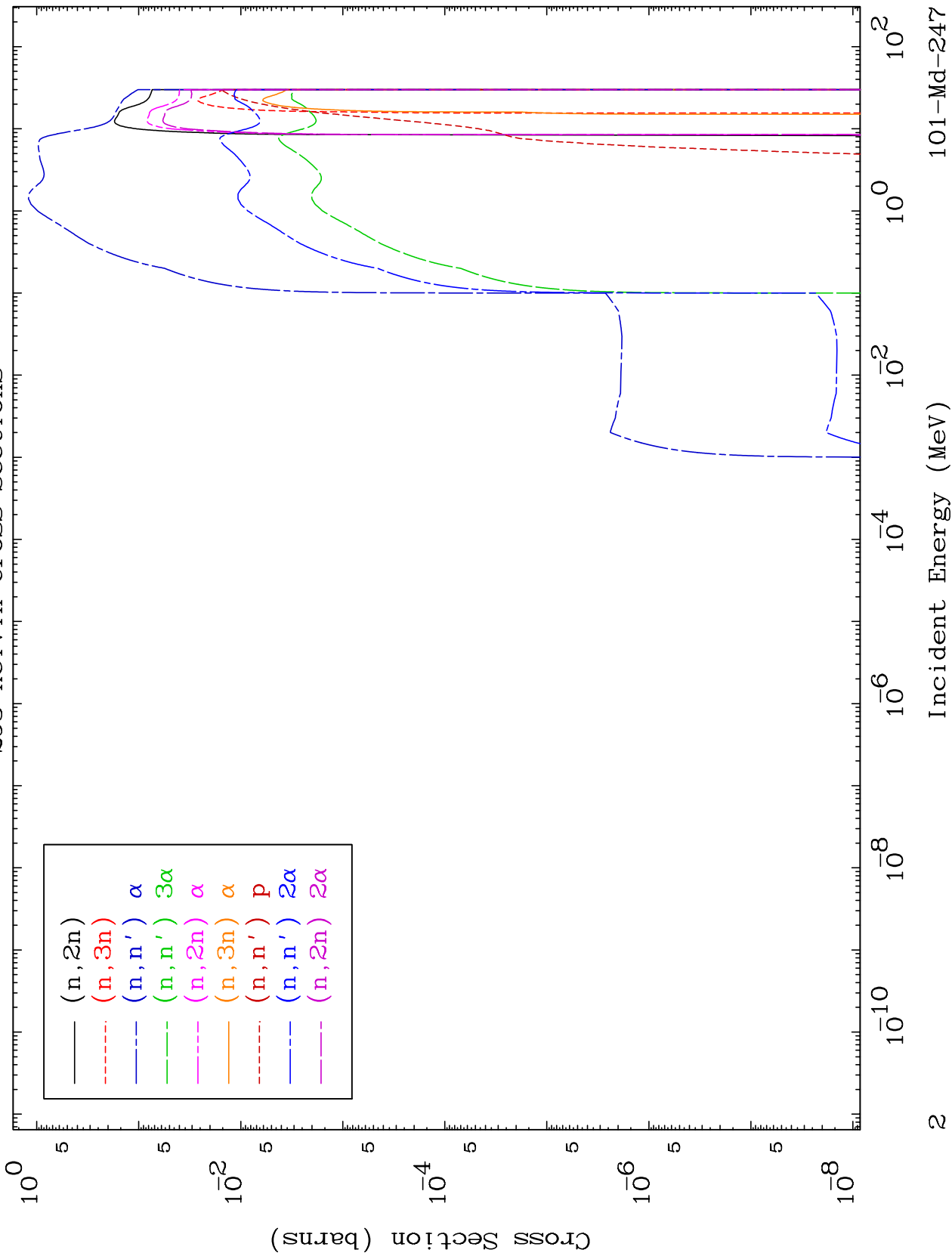
101-Md-247



MAT 9948

Neutron Production
293 Kelvin Cross Sections

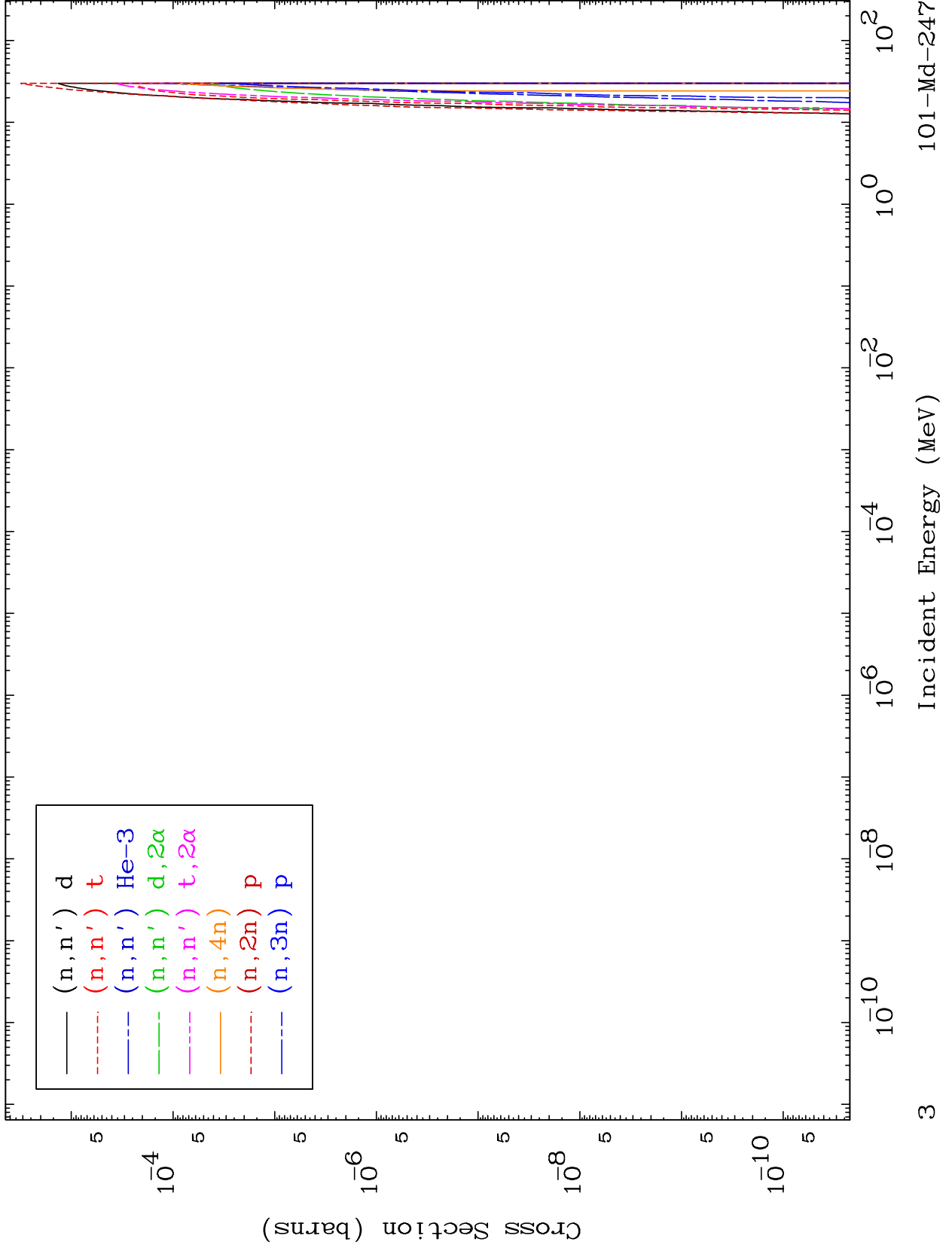
101-Md-247



MAT 9948

Neutron Production
293 Kelvin Cross Sections

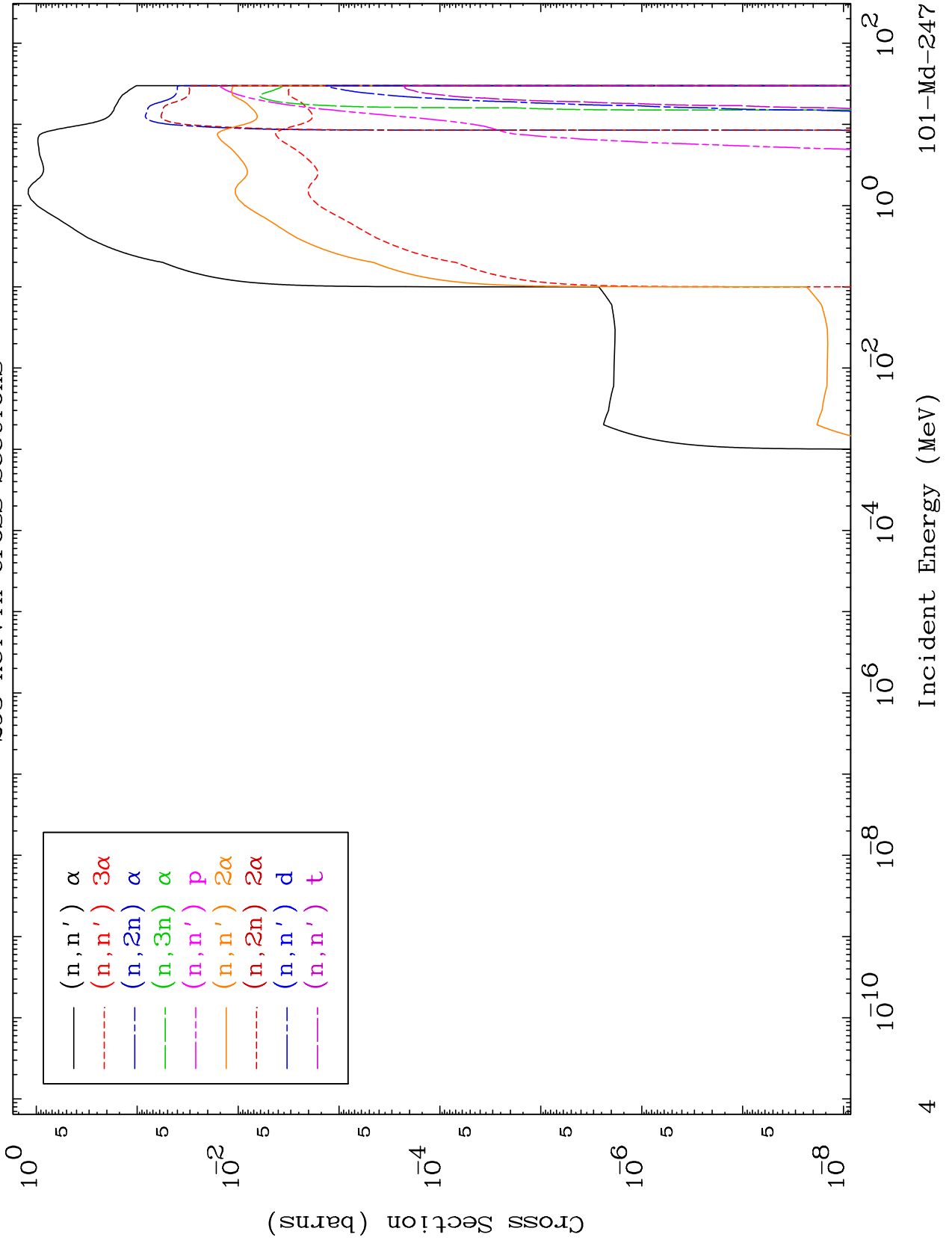
101-Md-247



MAT 9948

Charged Particle
293 Kelvin Cross Sections

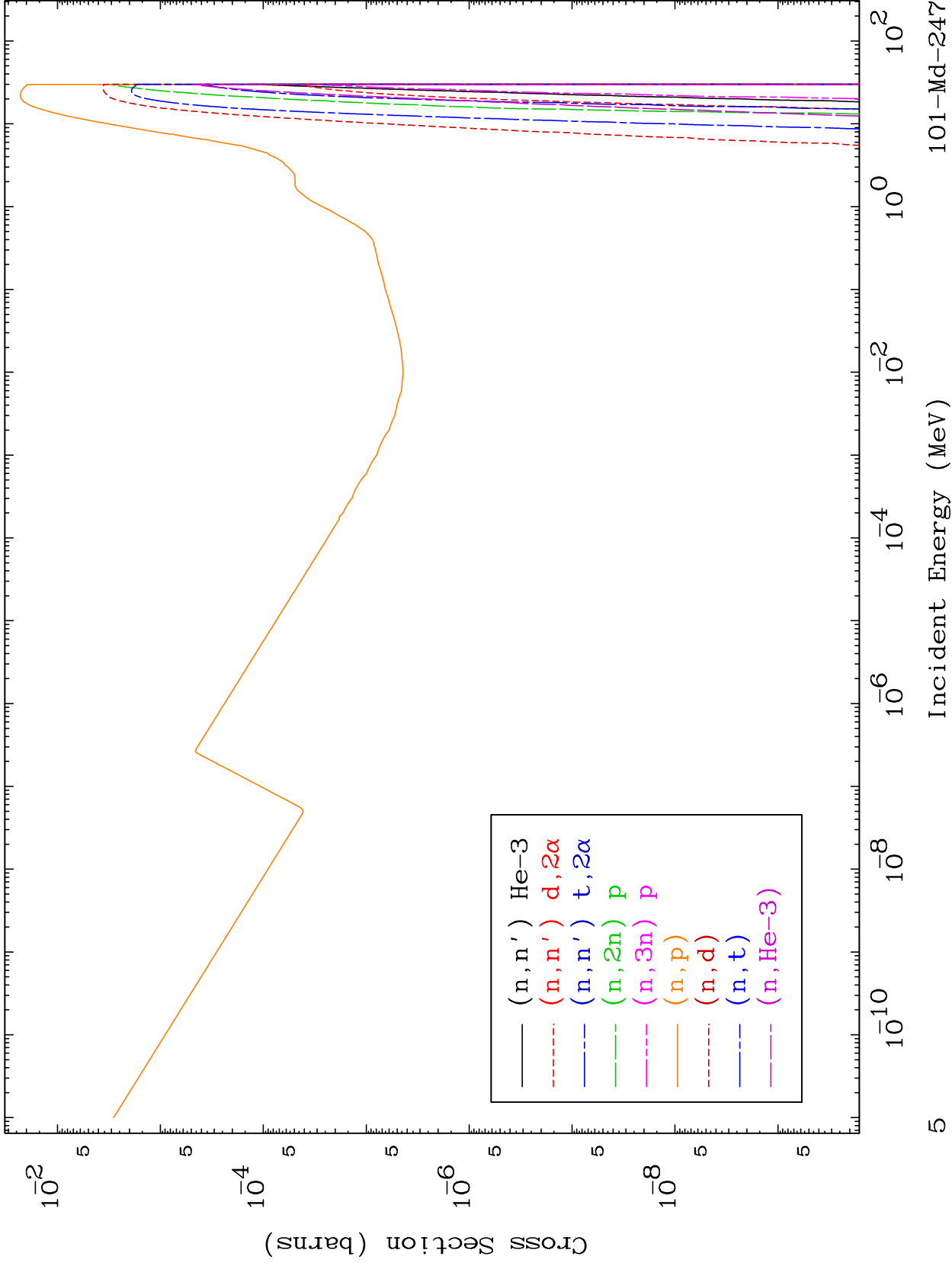
101-Md-247



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Charged Particle
293 Kelvin Cross Sections

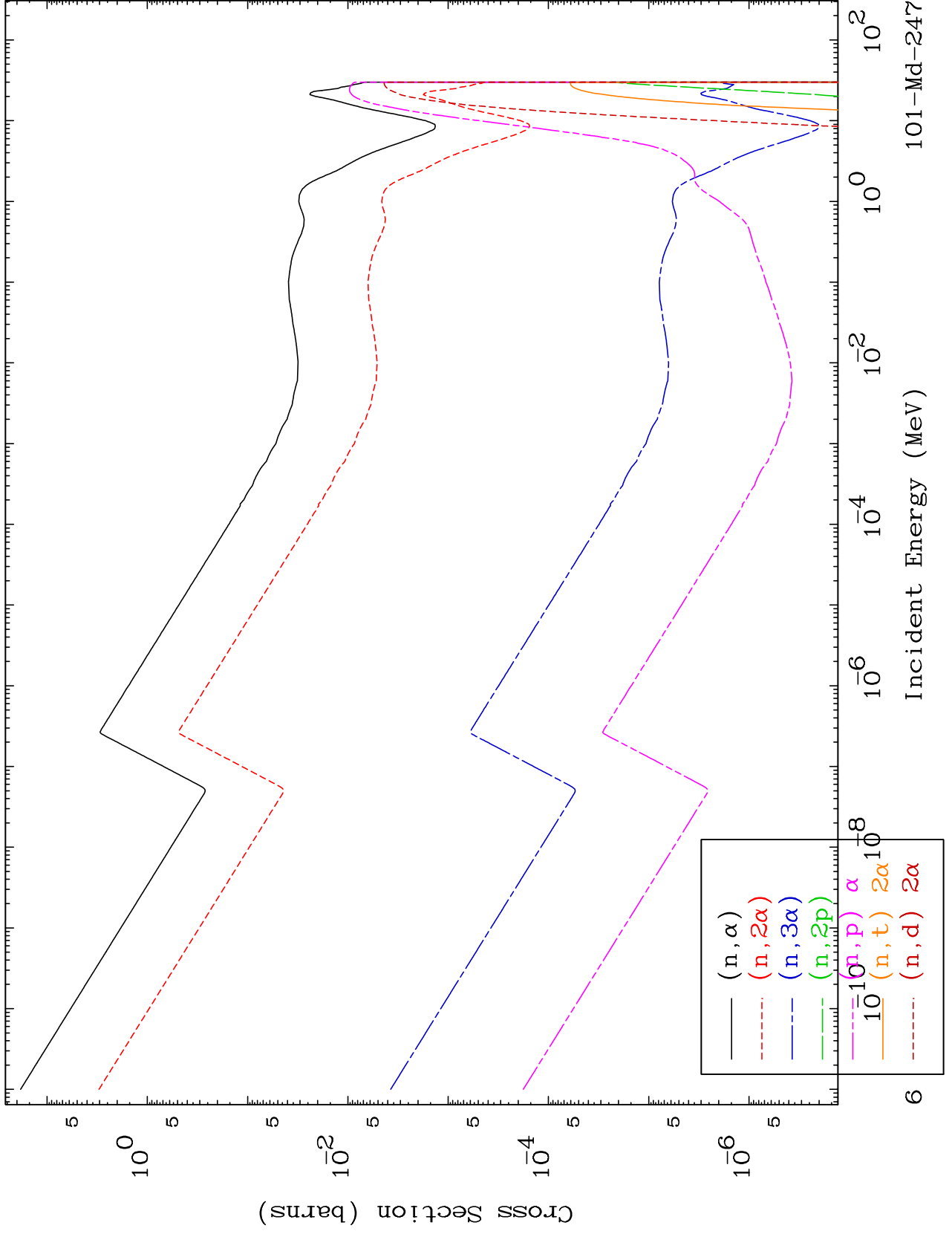
101-Md-247



MAT 9948

Charged Particle
293 Kelvin Cross Sections

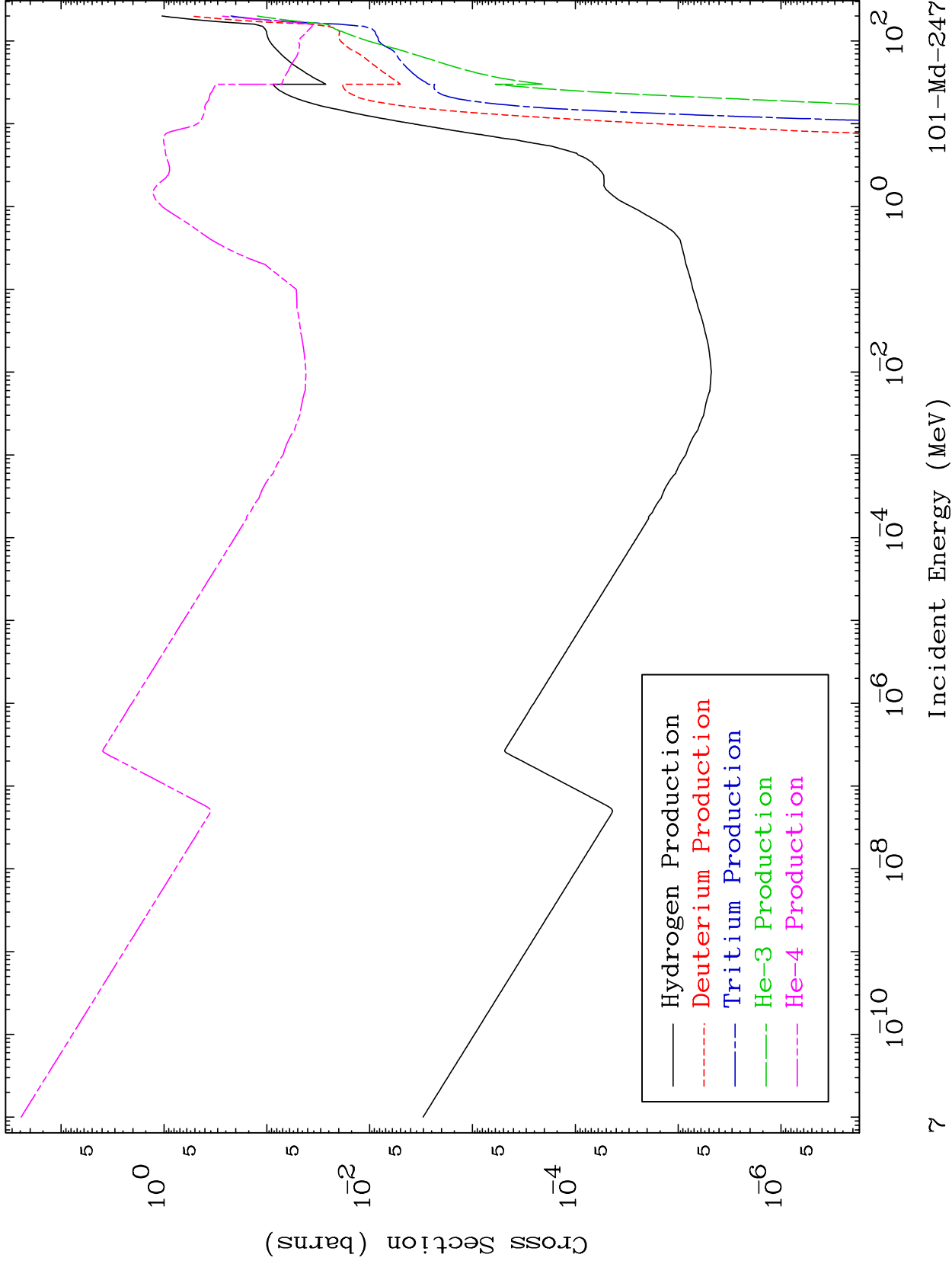
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Particle Production
293 Kelvin Cross Sections

101-Md-247

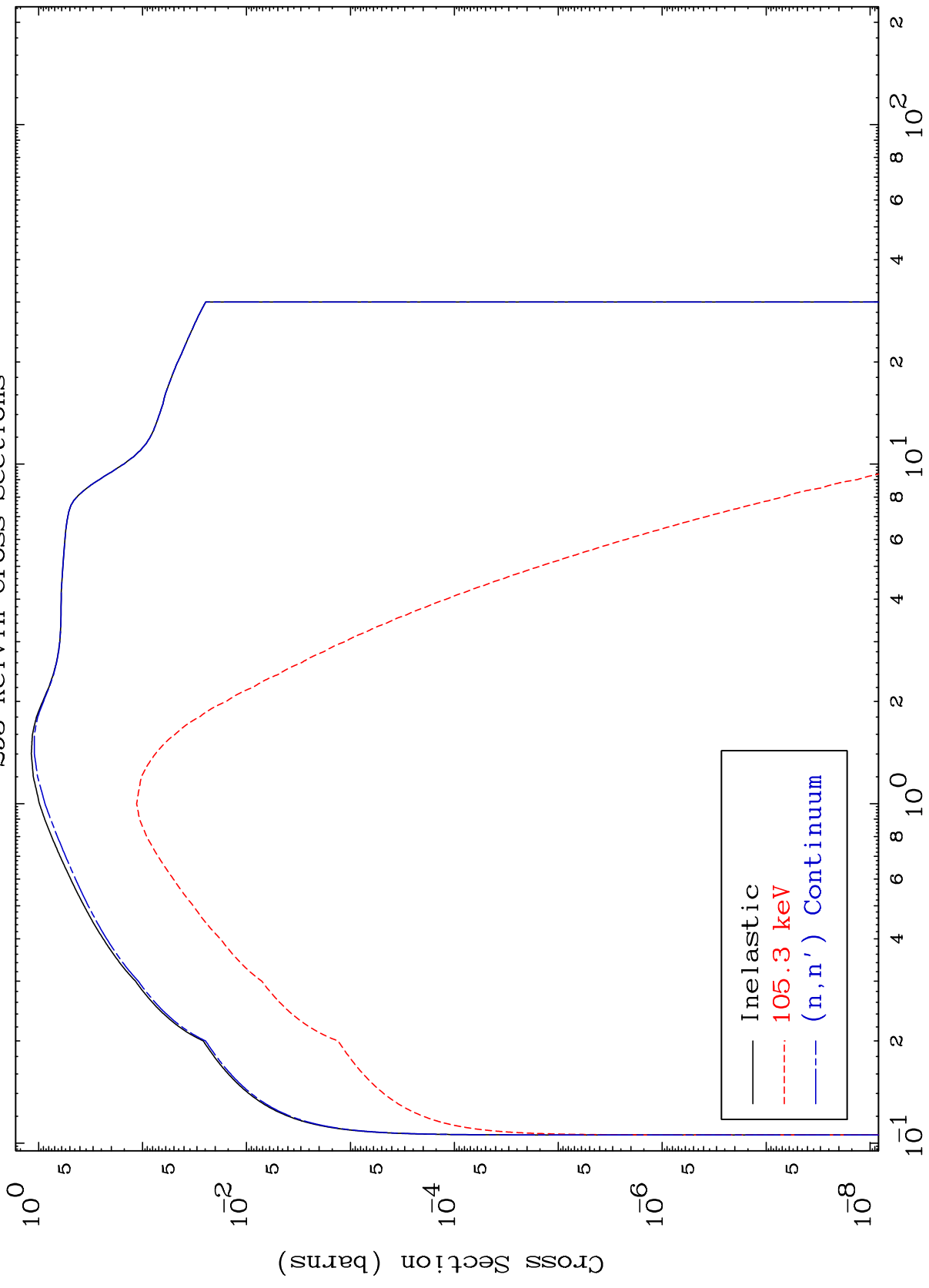


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(n,n') Level

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293 Kelvin Cross Sections



Incident Energy (MeV)

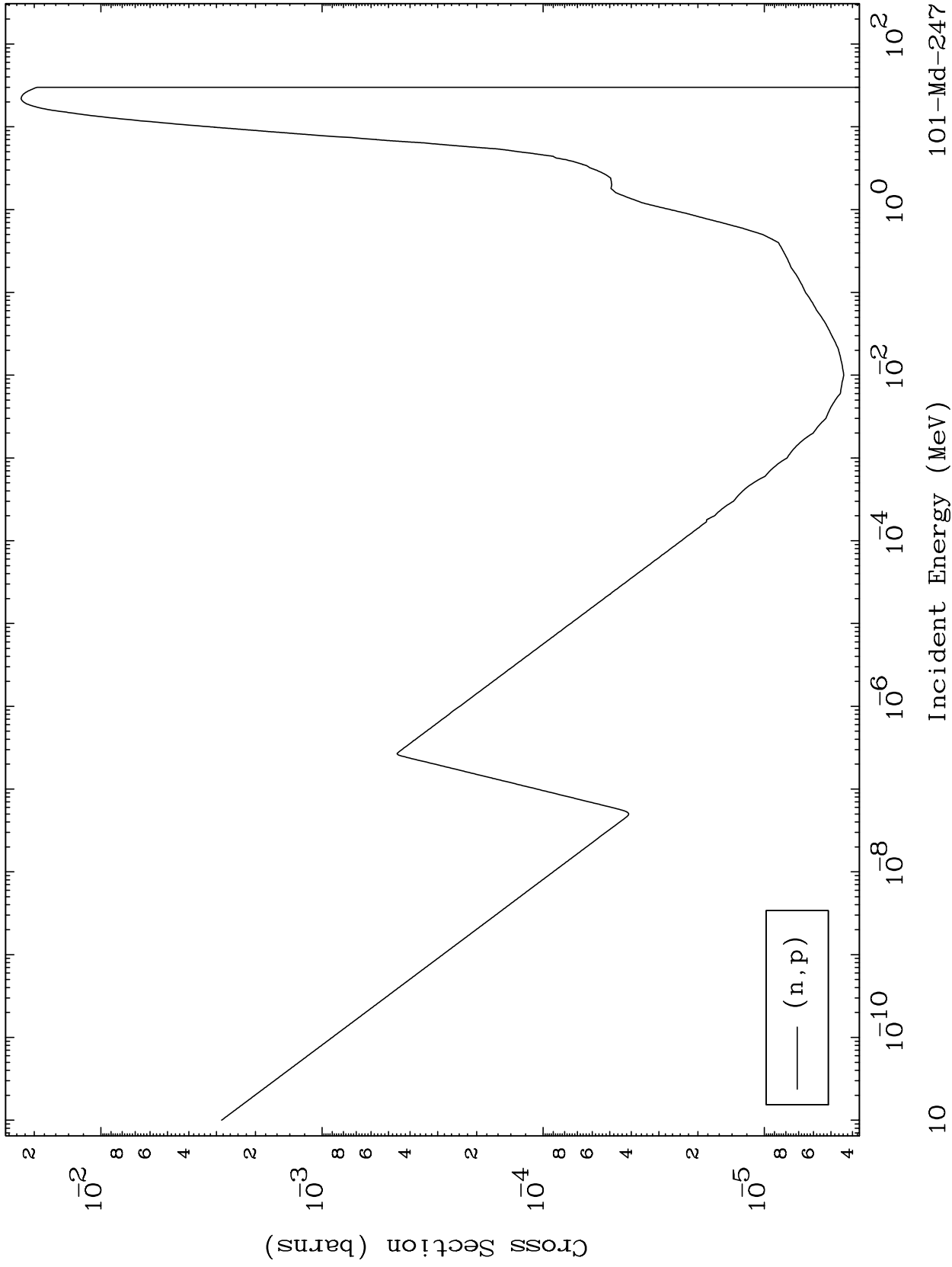
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(n,p) Levels
293 Kelvin Cross Sections

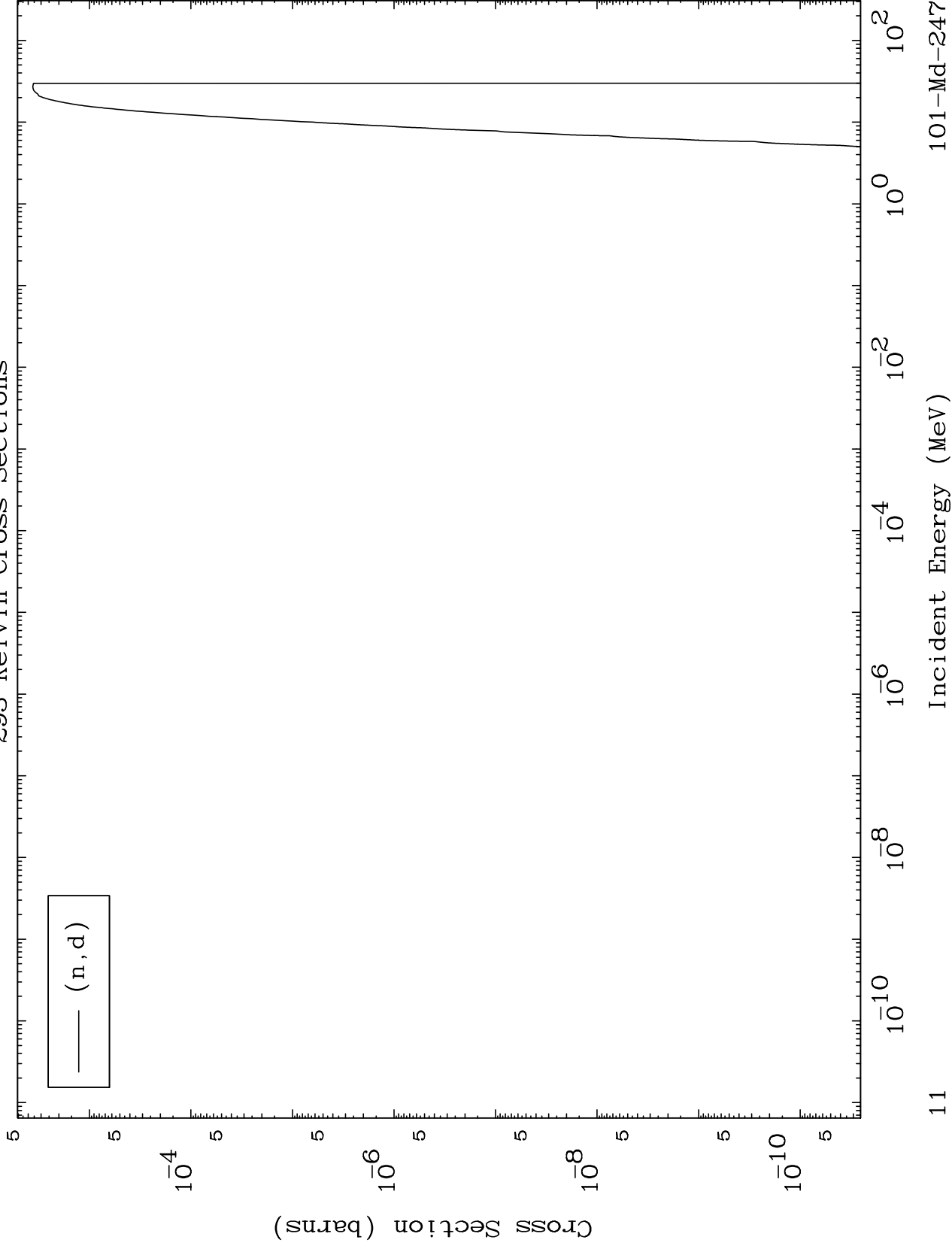
101-Md-247



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(n,d) Levels
293 Kelvin Cross Sections

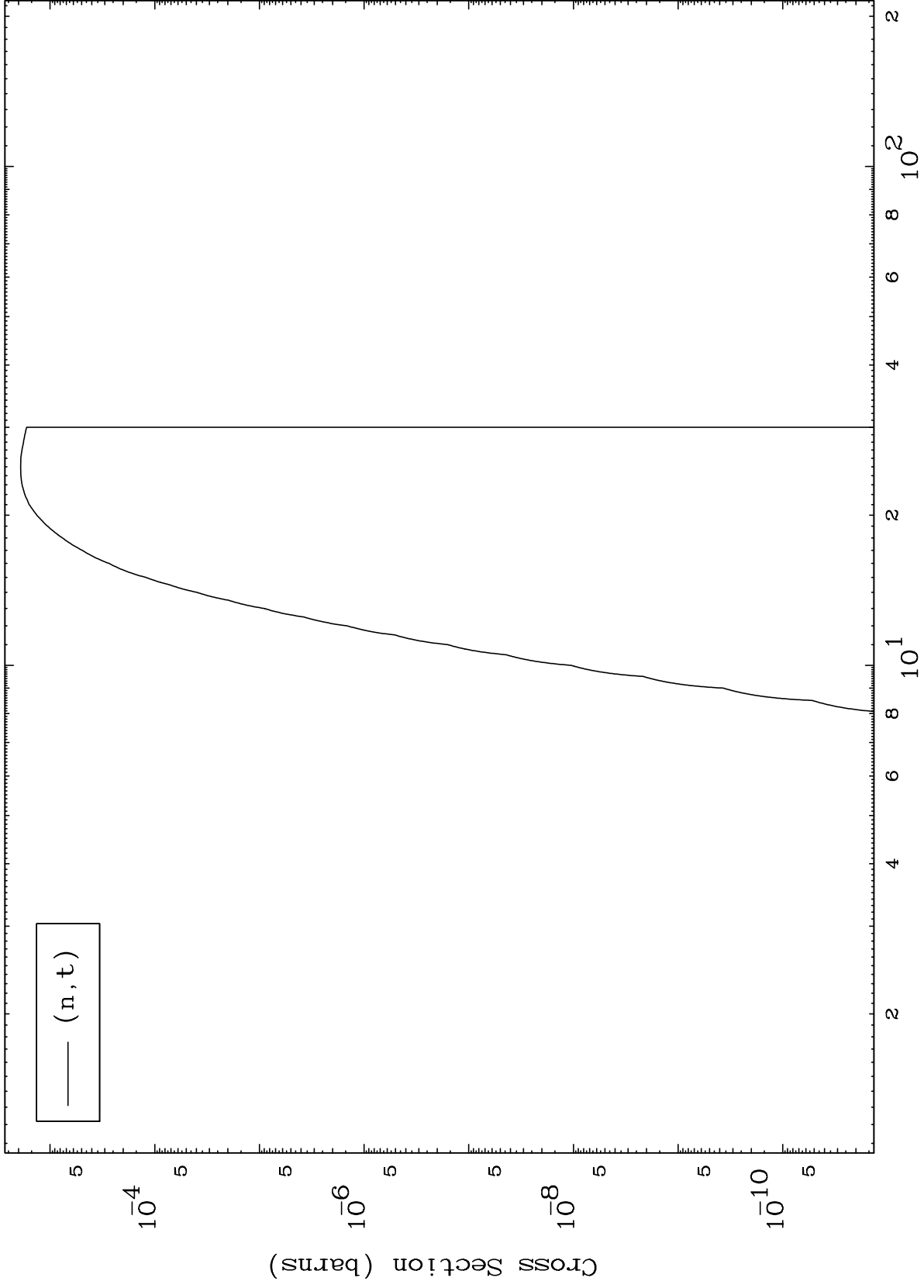
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(n,t) Levels
293 Kelvin Cross Sections

101-Md-247



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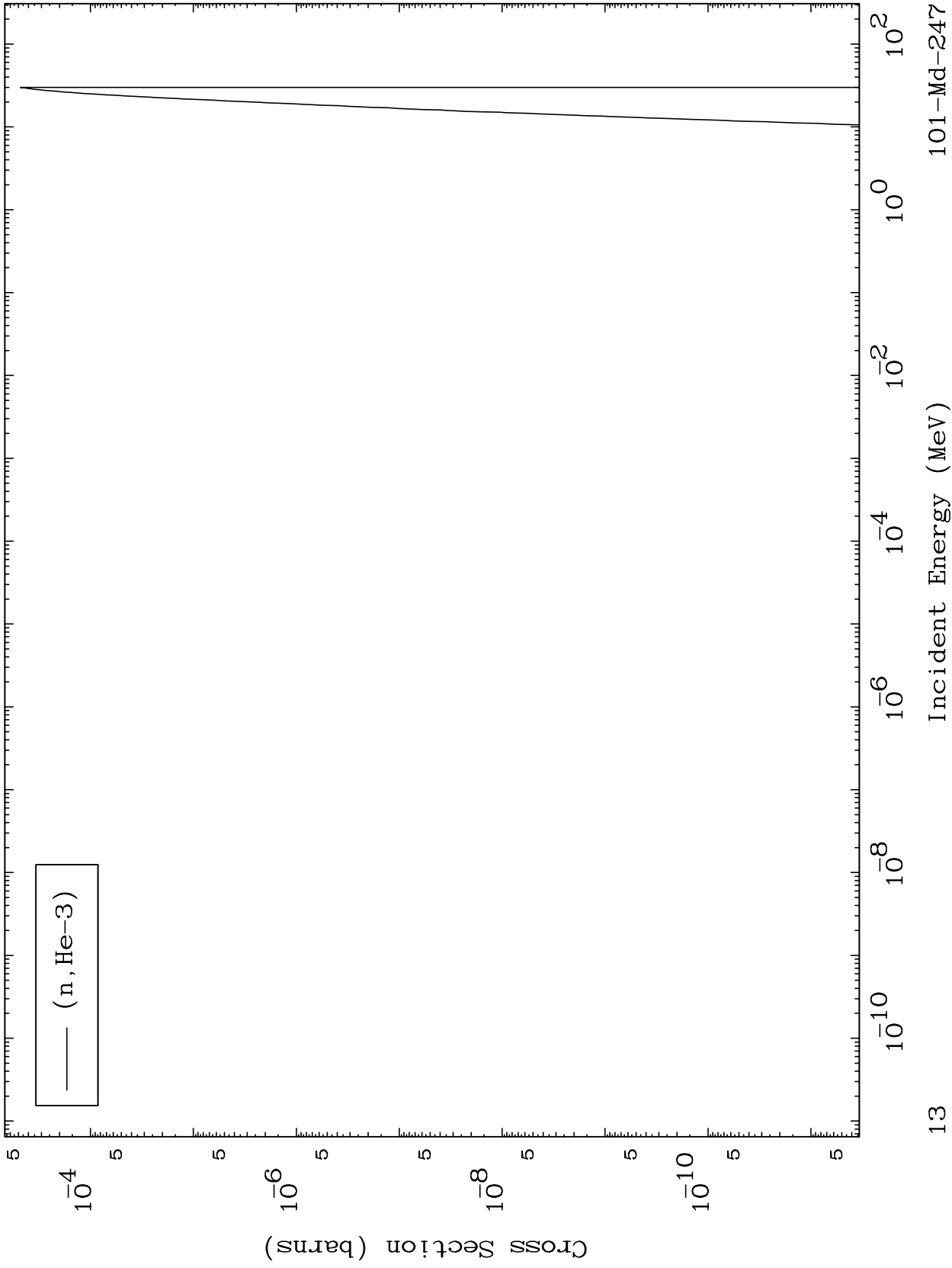
Incident Energy (MeV)

101-Md-247

MAT 9948

(n,He3) Levels
293 Kelvin Cross Sections

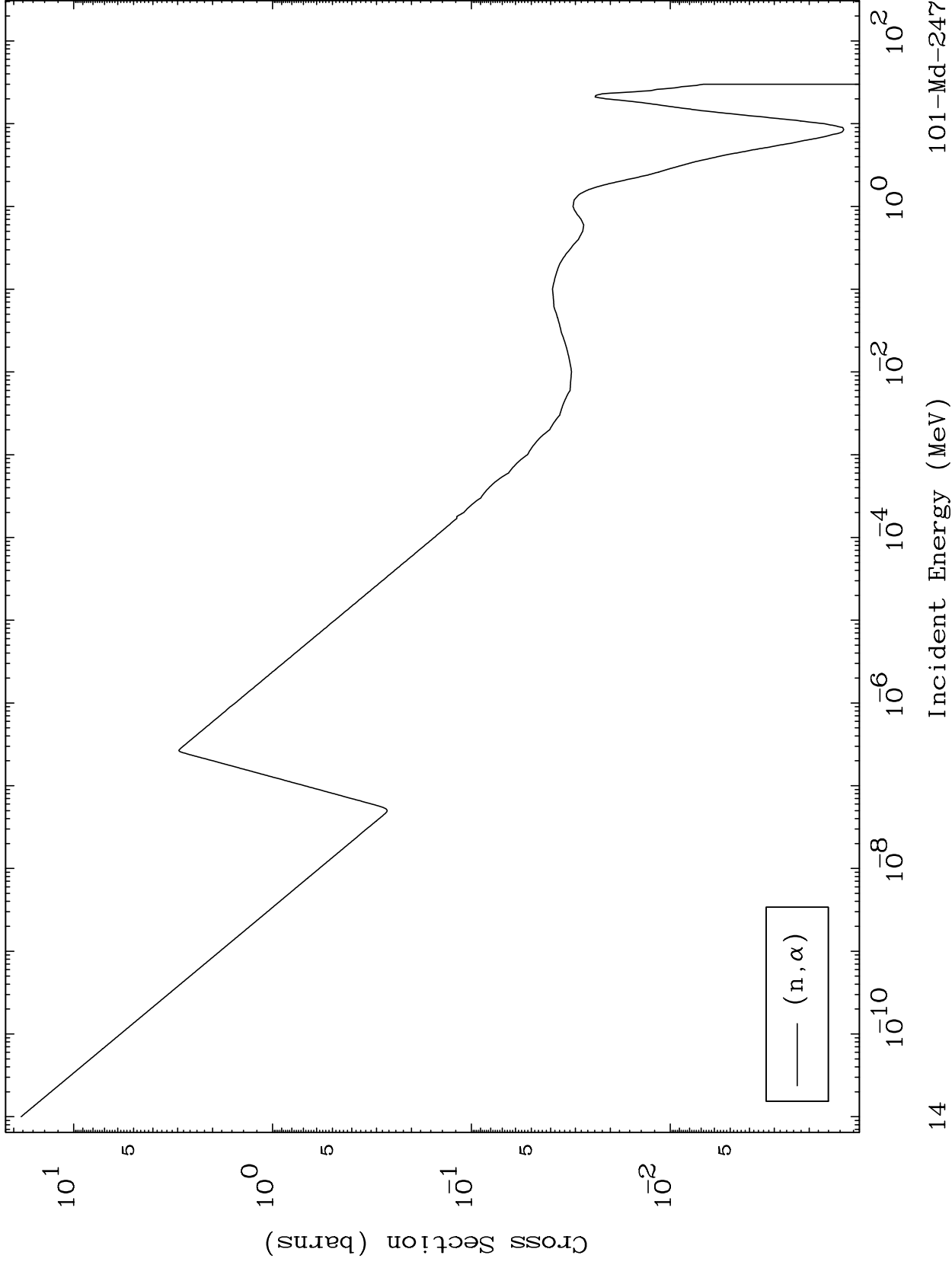
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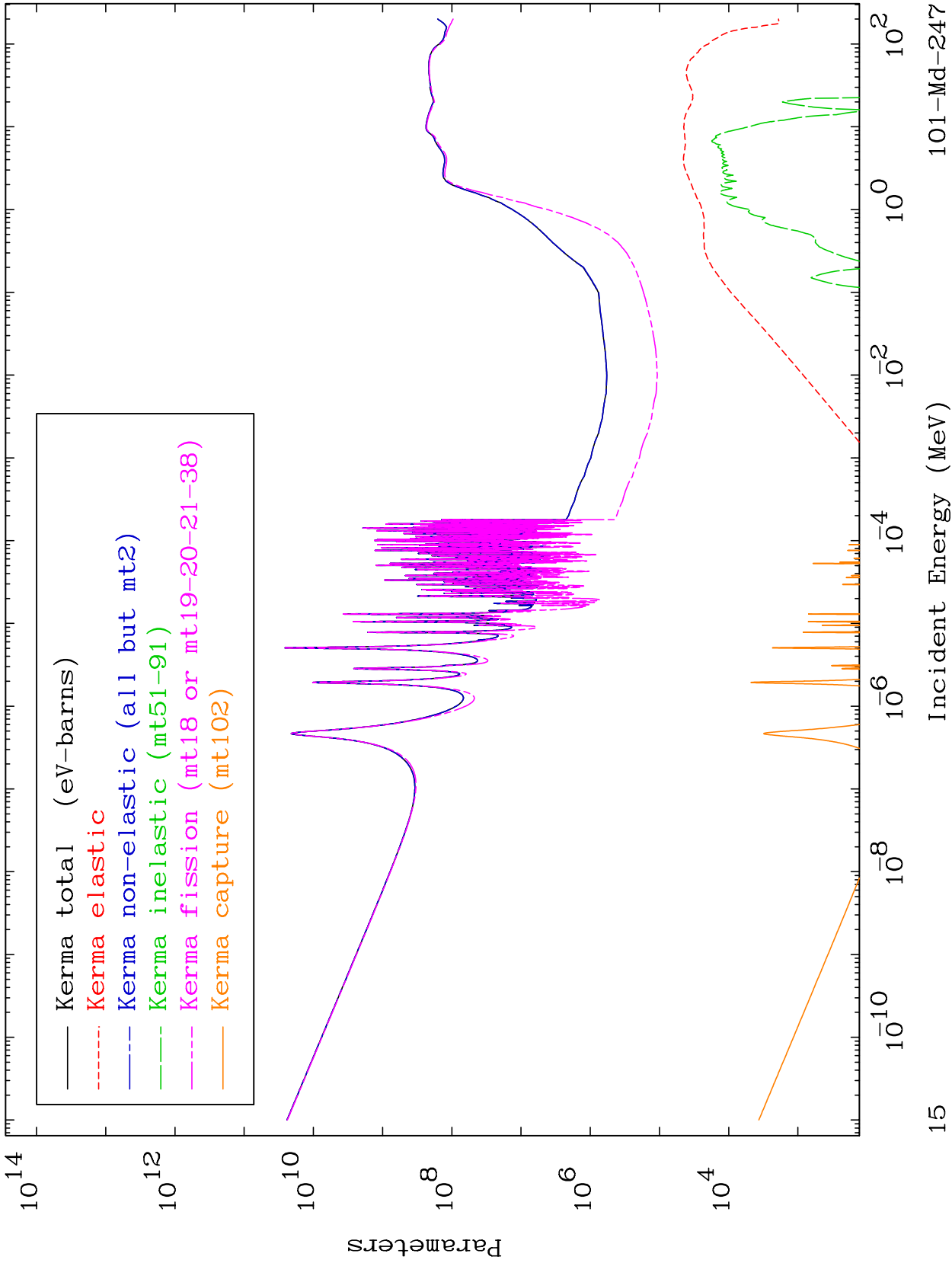


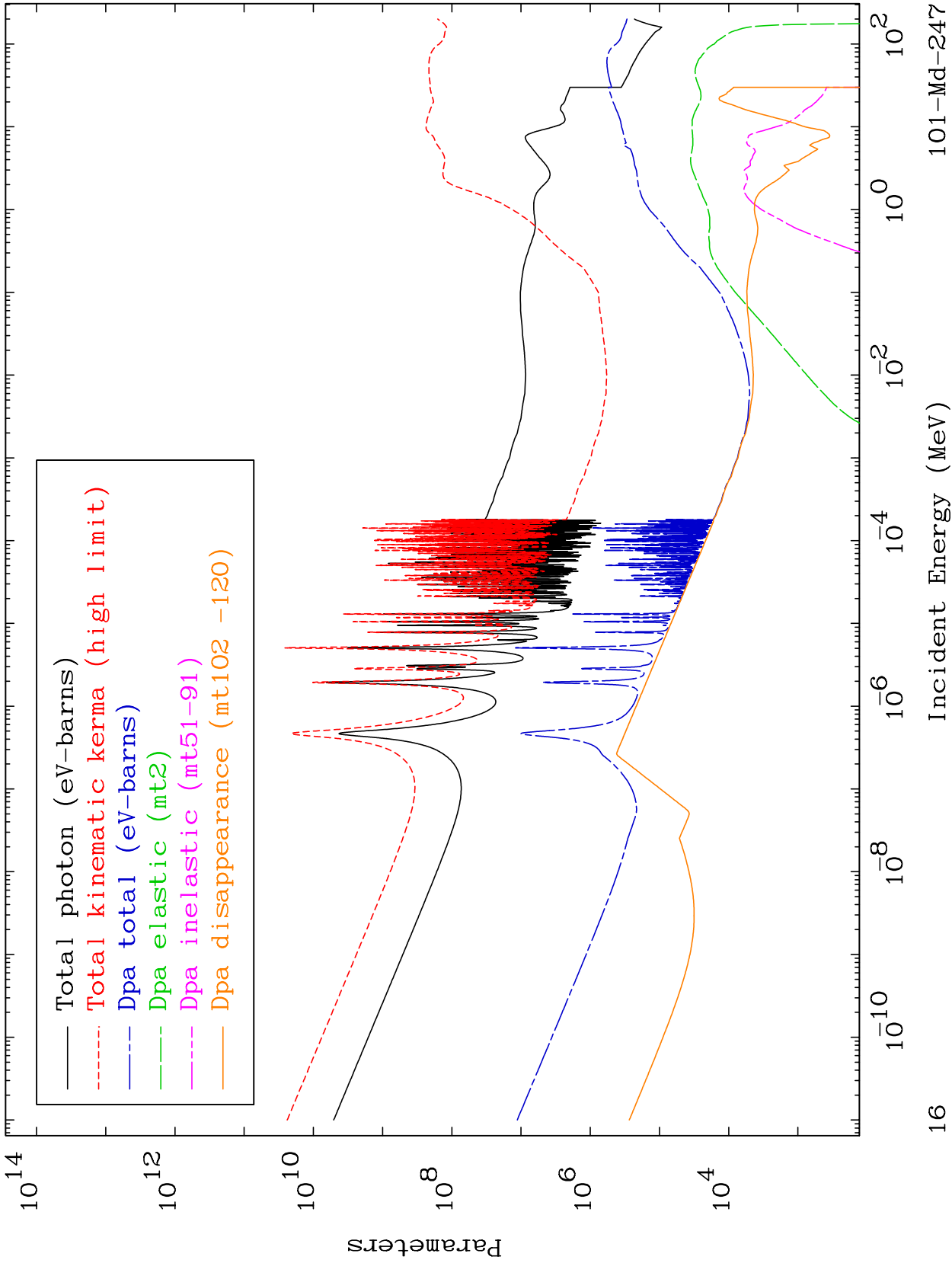
MAT 9948

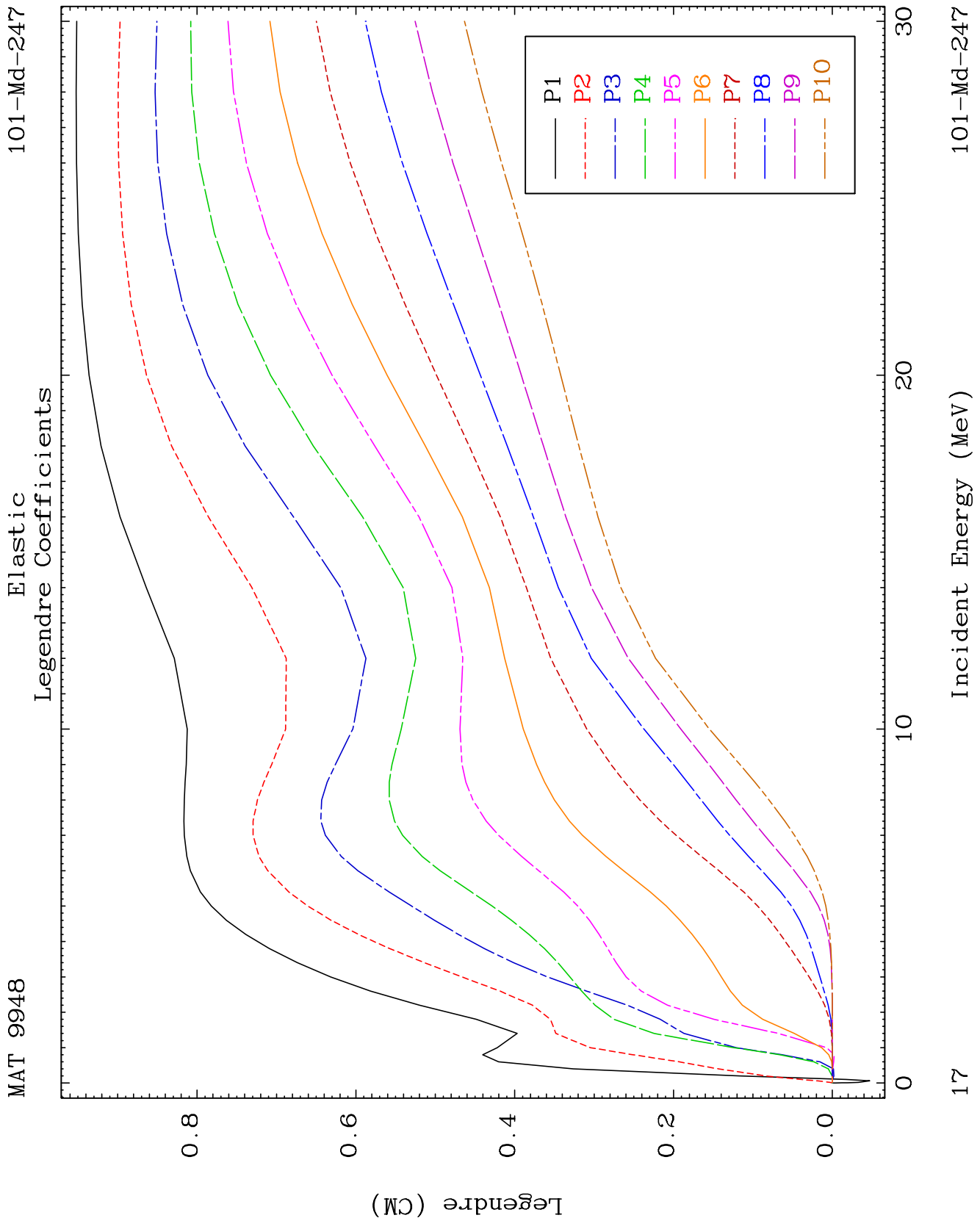
(n, α) Levels
293 Kelvin Cross Sections

101-Md-247









101-Md-247

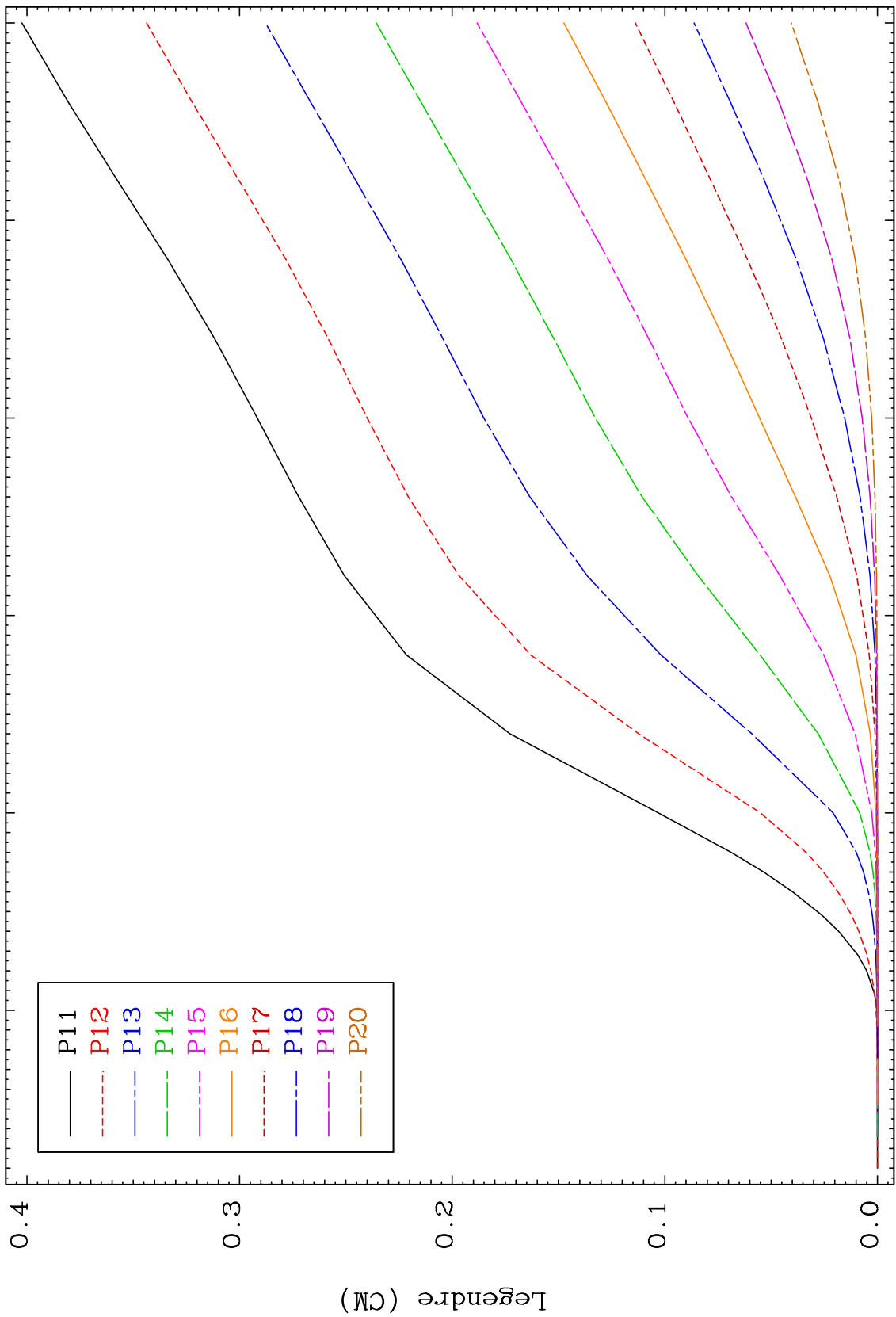
Incident Energy (MeV)

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Elastic Legendre Coefficients

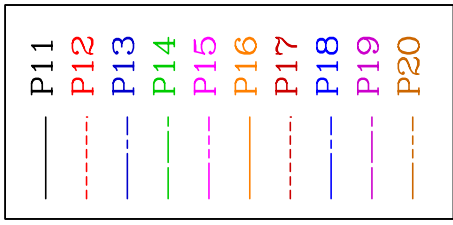
101-Md-247



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Incident Energy (MeV)

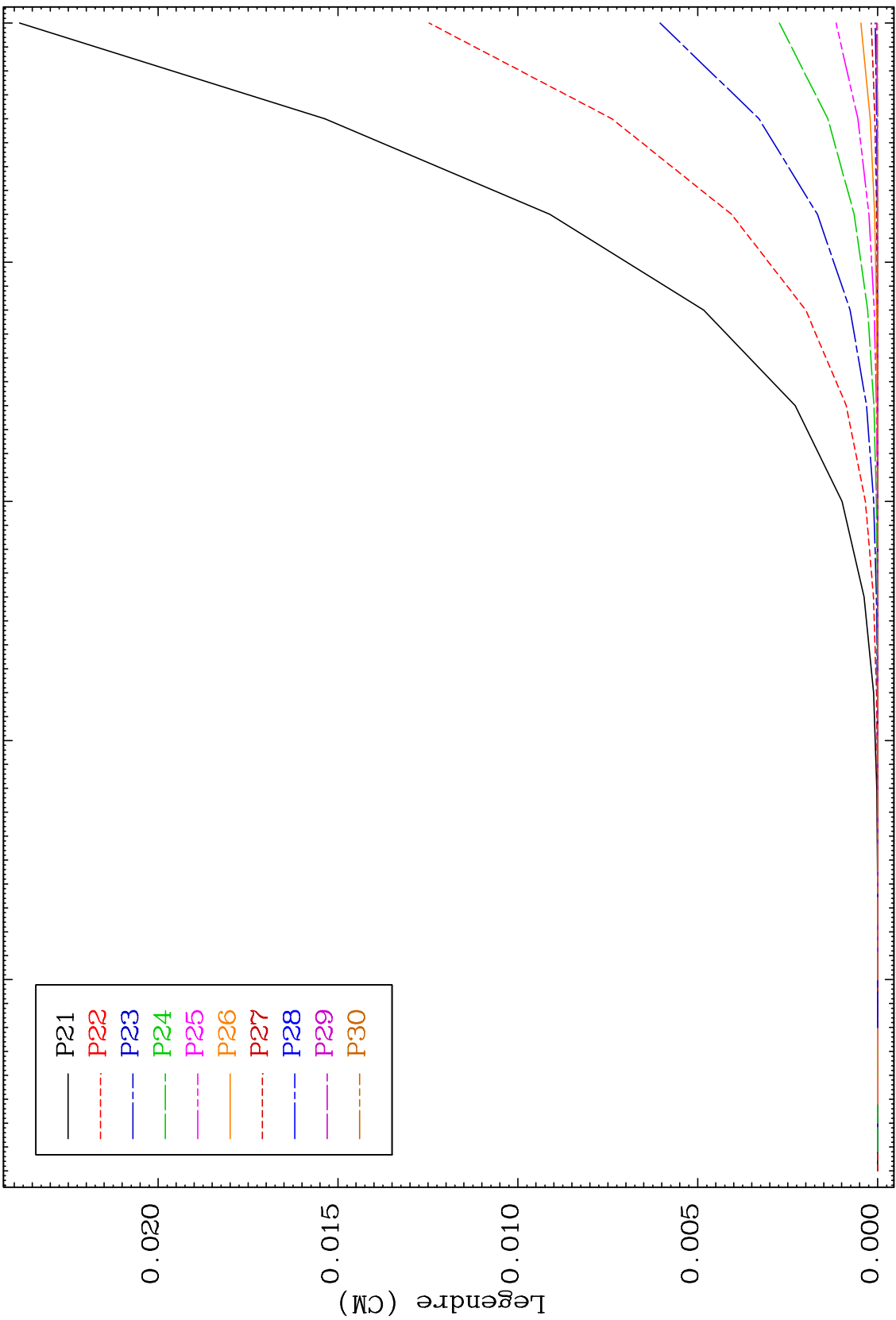
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Elastic Legendre Coefficients

101-Md-247



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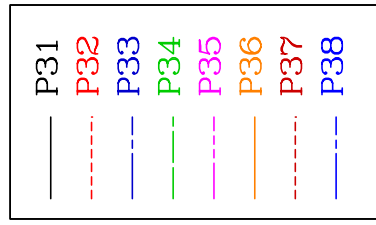
Incident Energy (MeV)

101-Md-247

MAT 9948

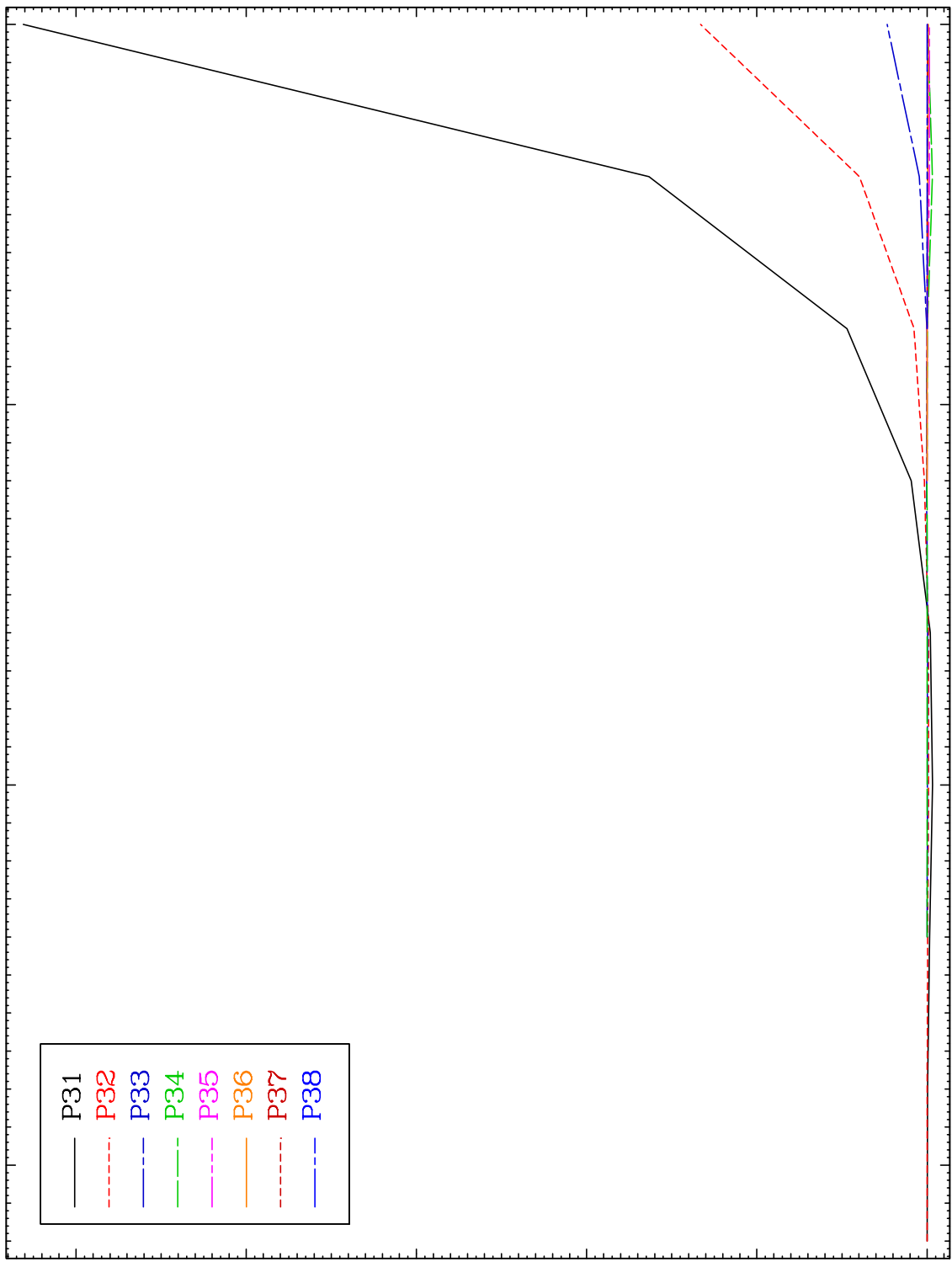
Elastic Legendre Coefficients

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$\times 10^{-6}$

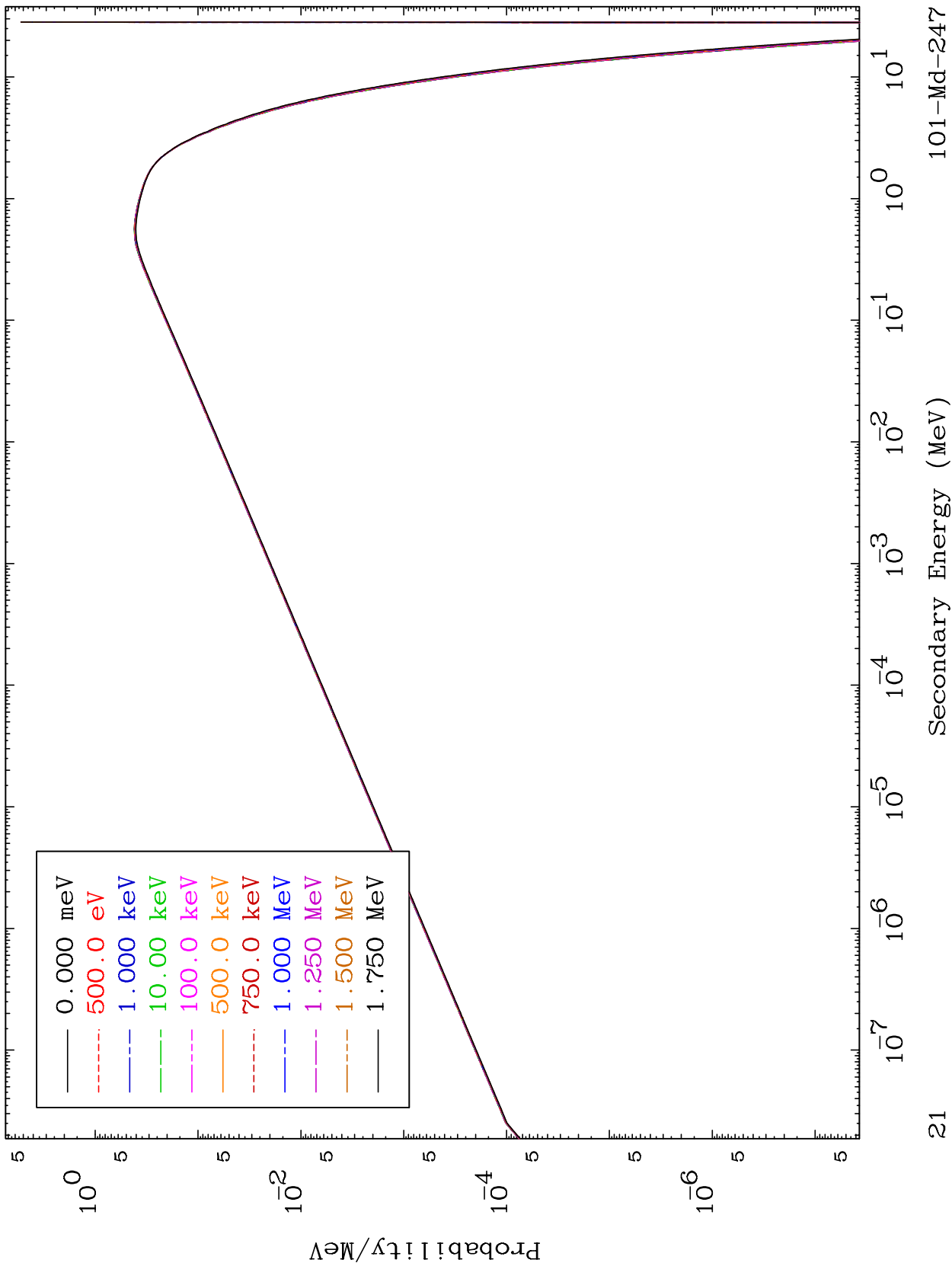
Legendre (CM)



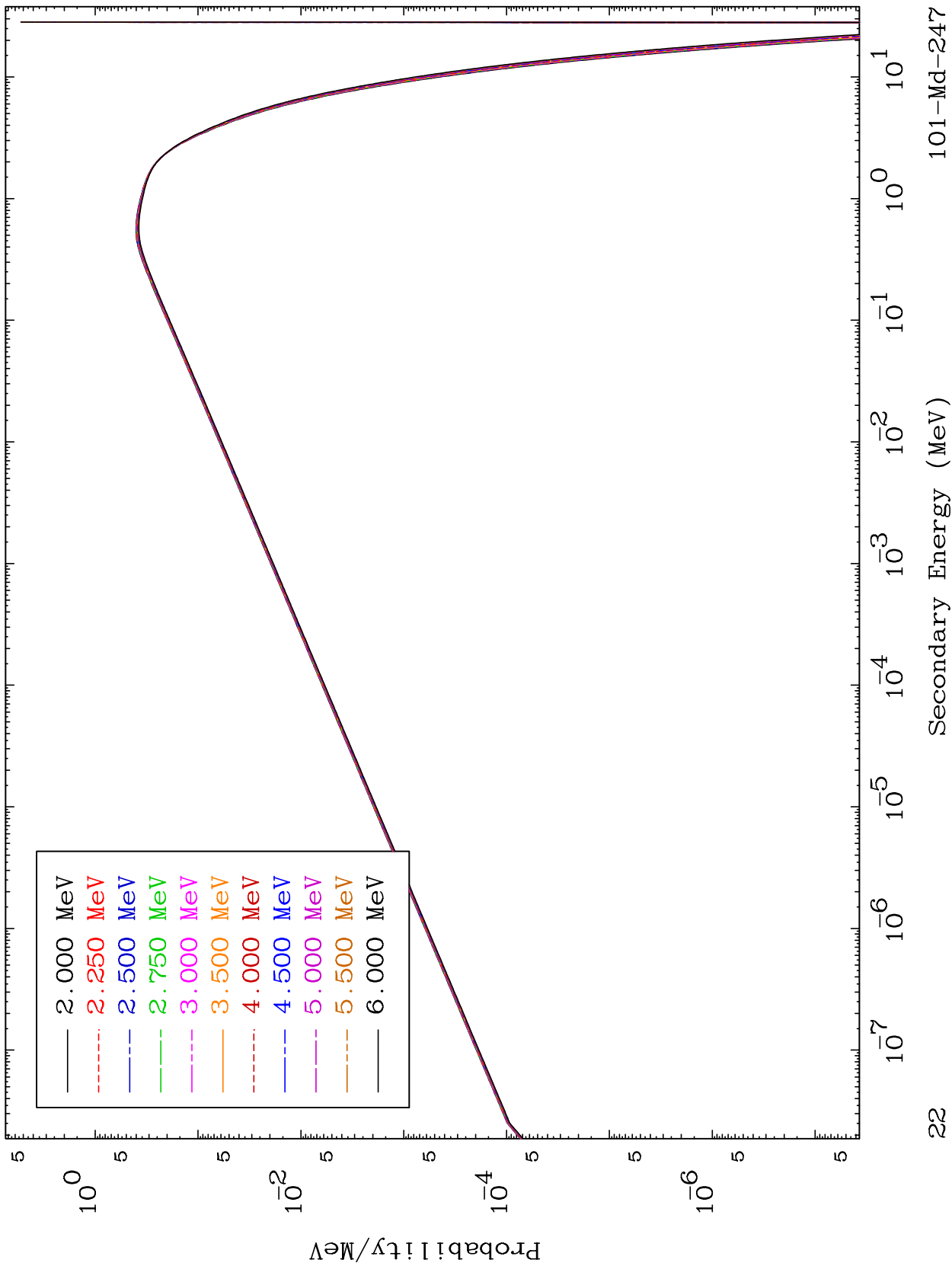
20

Incident Energy (MeV)

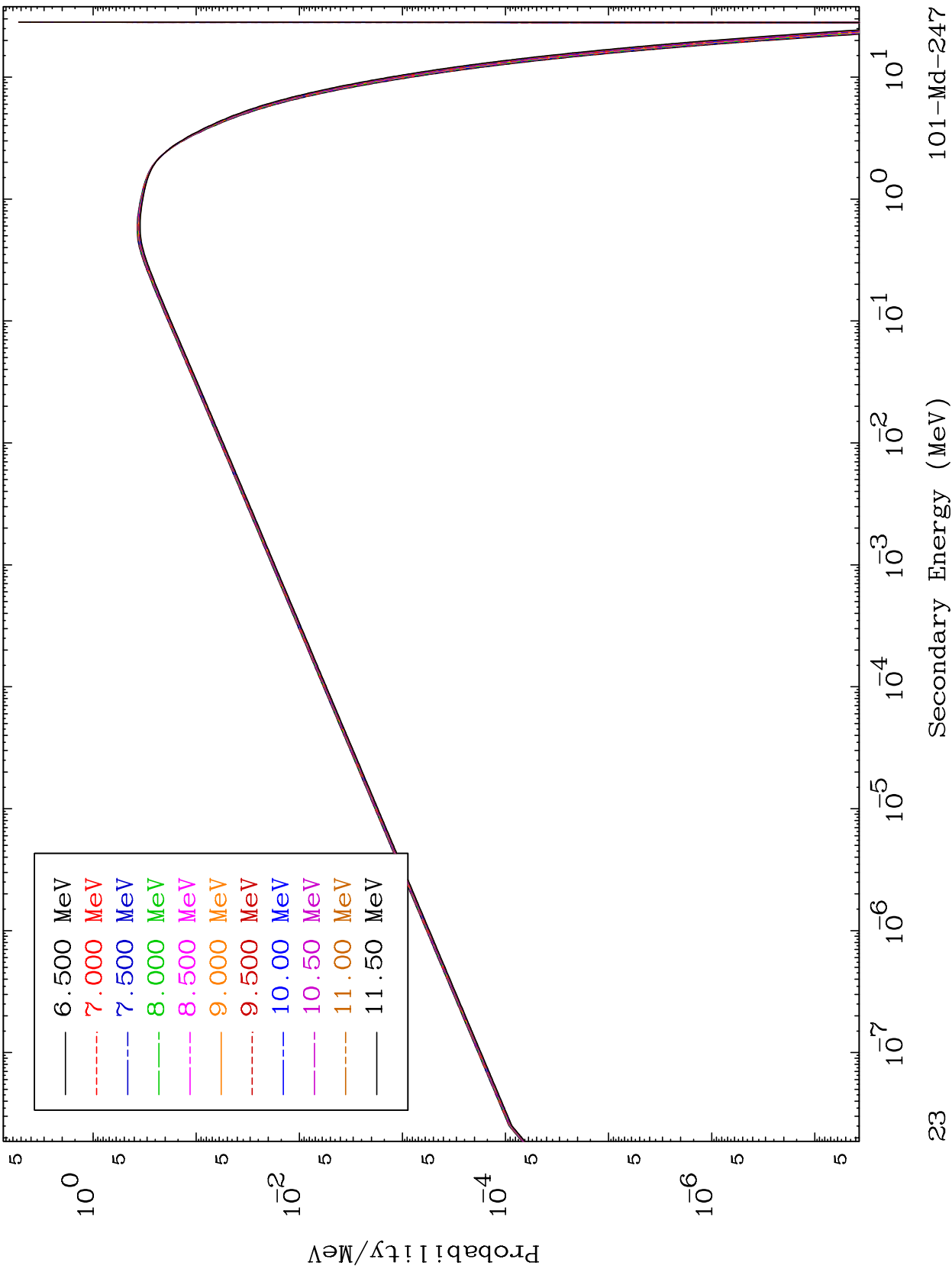
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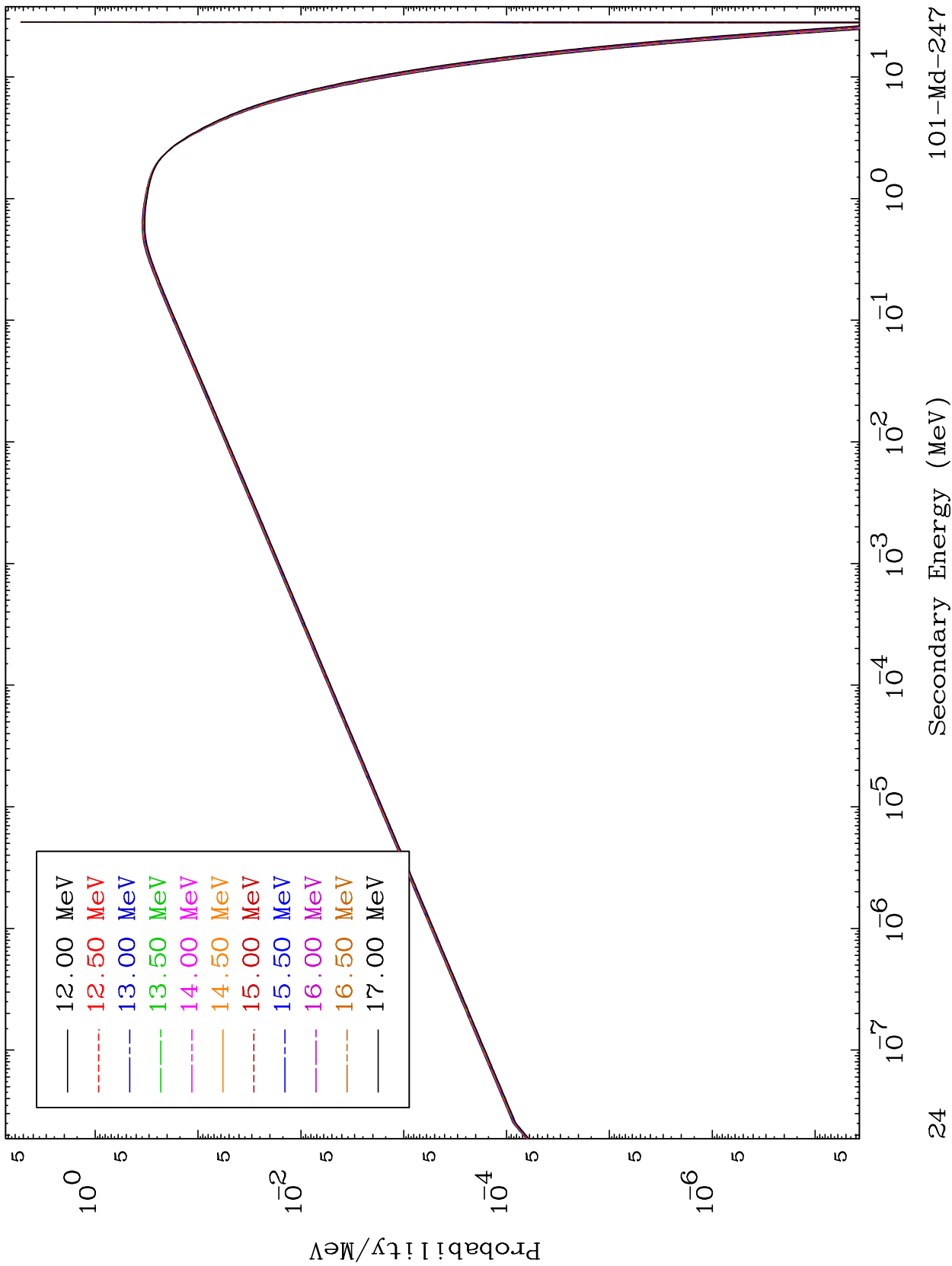


Fission
Energy Distribution



Fission
Energy Distribution

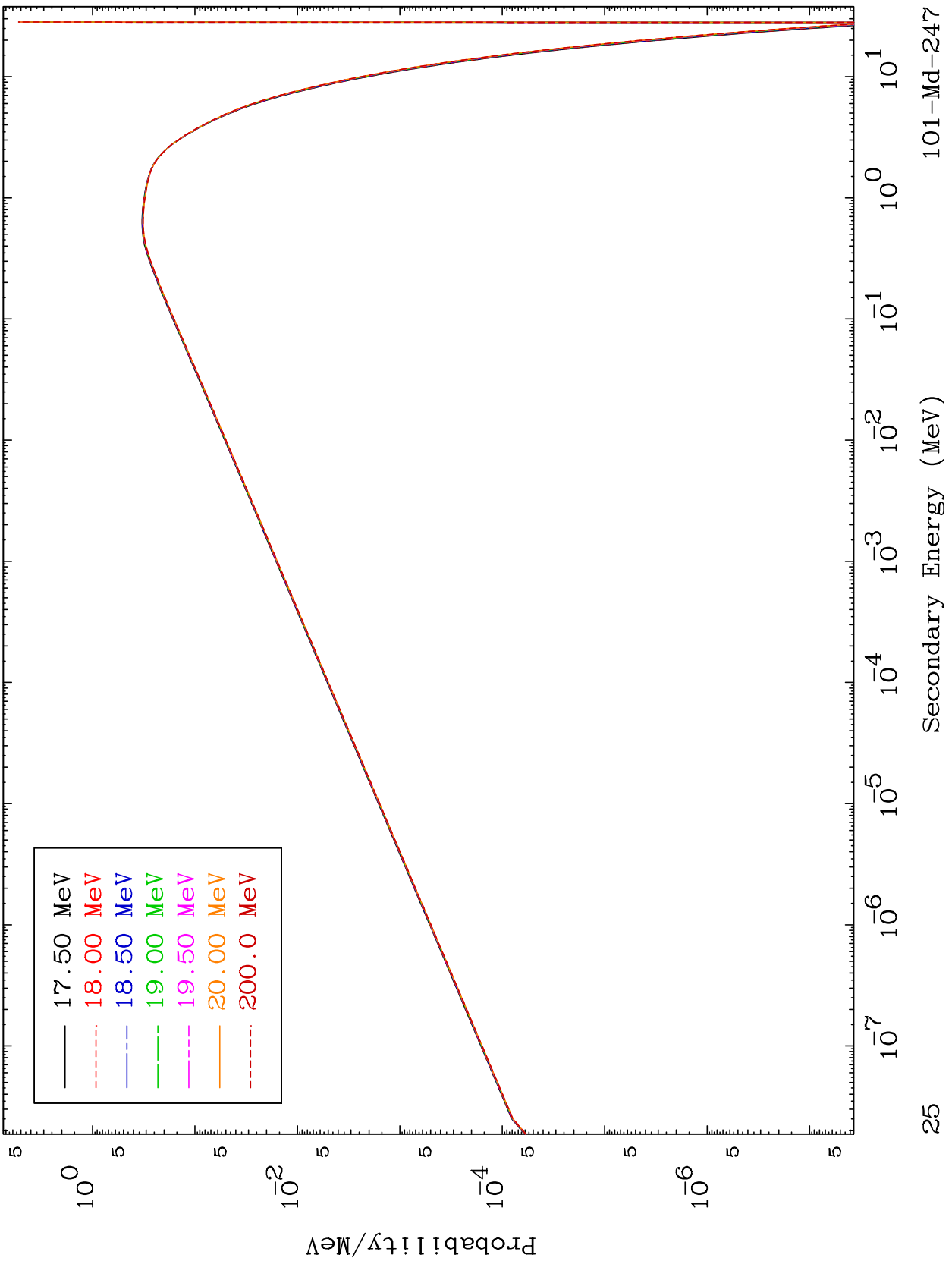




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Fission
Energy Distribution

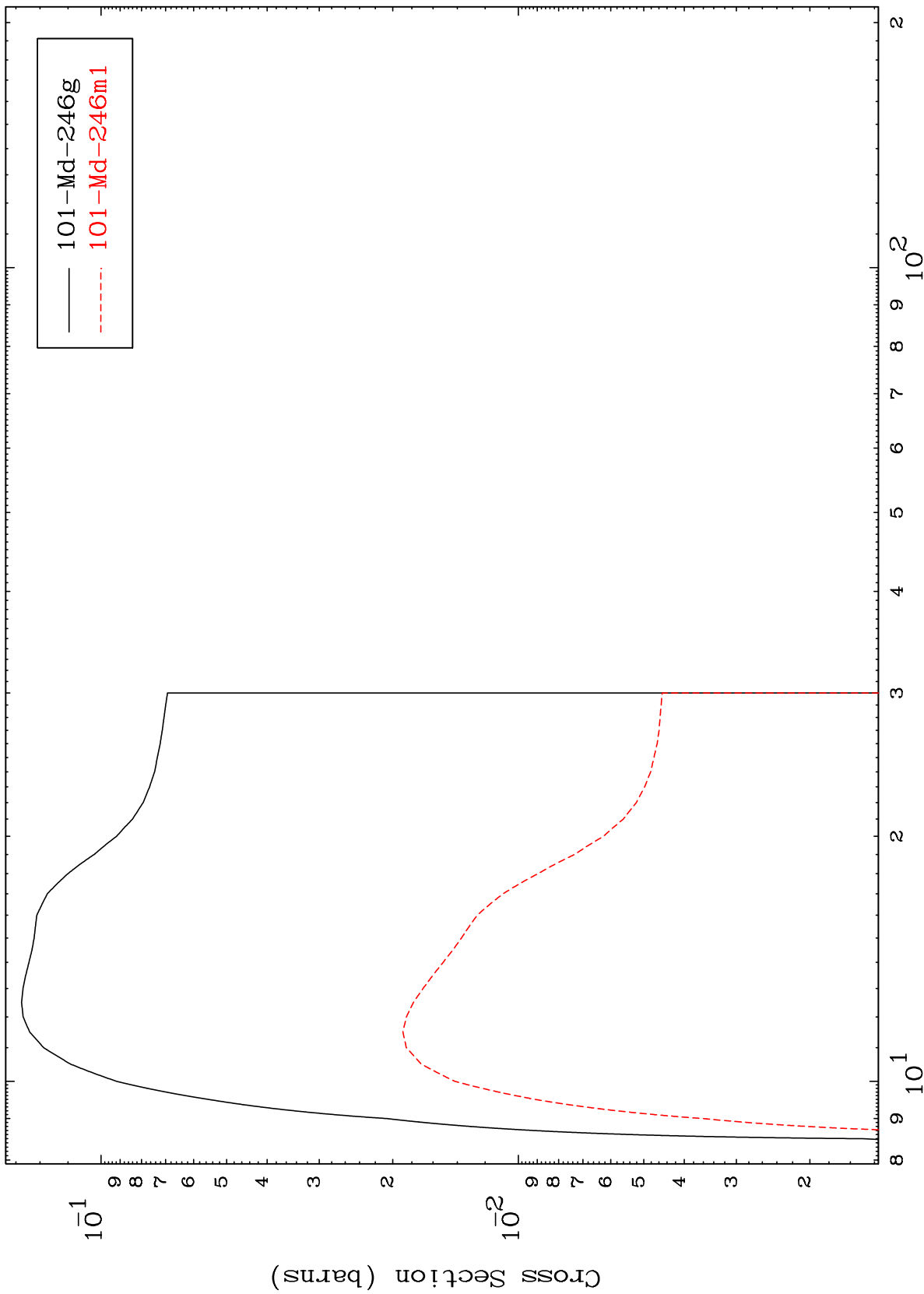
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101-Md-247

(n,2n)
Radionuclide Production Cross Section



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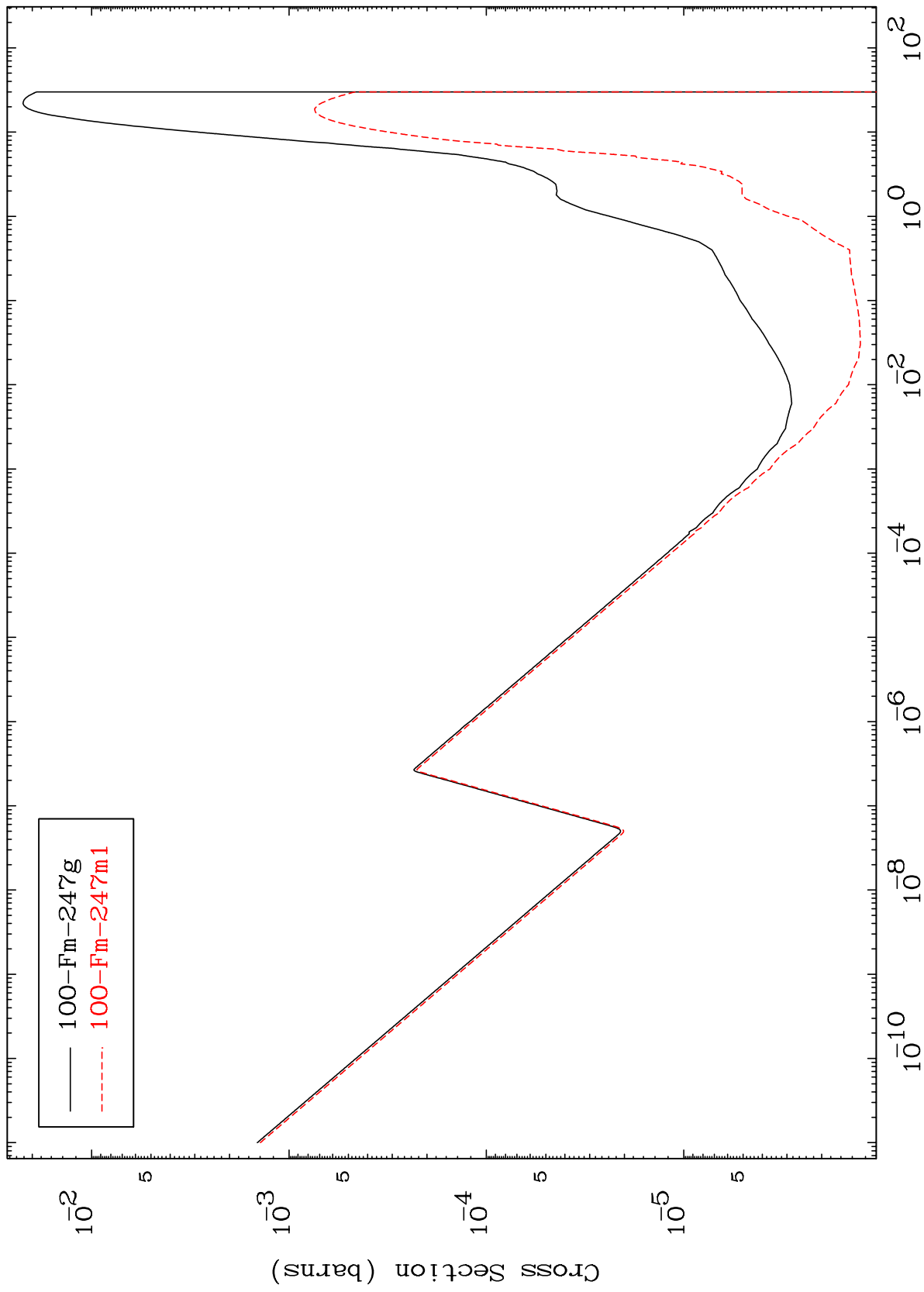
Incident Energy (MeV)

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(n,p)
Radionuclide Production Cross Section



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MAT 9948

101-Md-247

Radionuclide Production Cross Section
(n, 3 α)

