

Program EVALPLOT
(Version 2015-2)

by

Dermott E. Cullen
(Present Contact Information)

Dermott E. Cullen
1466 Hudson Way
Livermore, CA 94550
U.S.A.

Tele: 925-443-1911

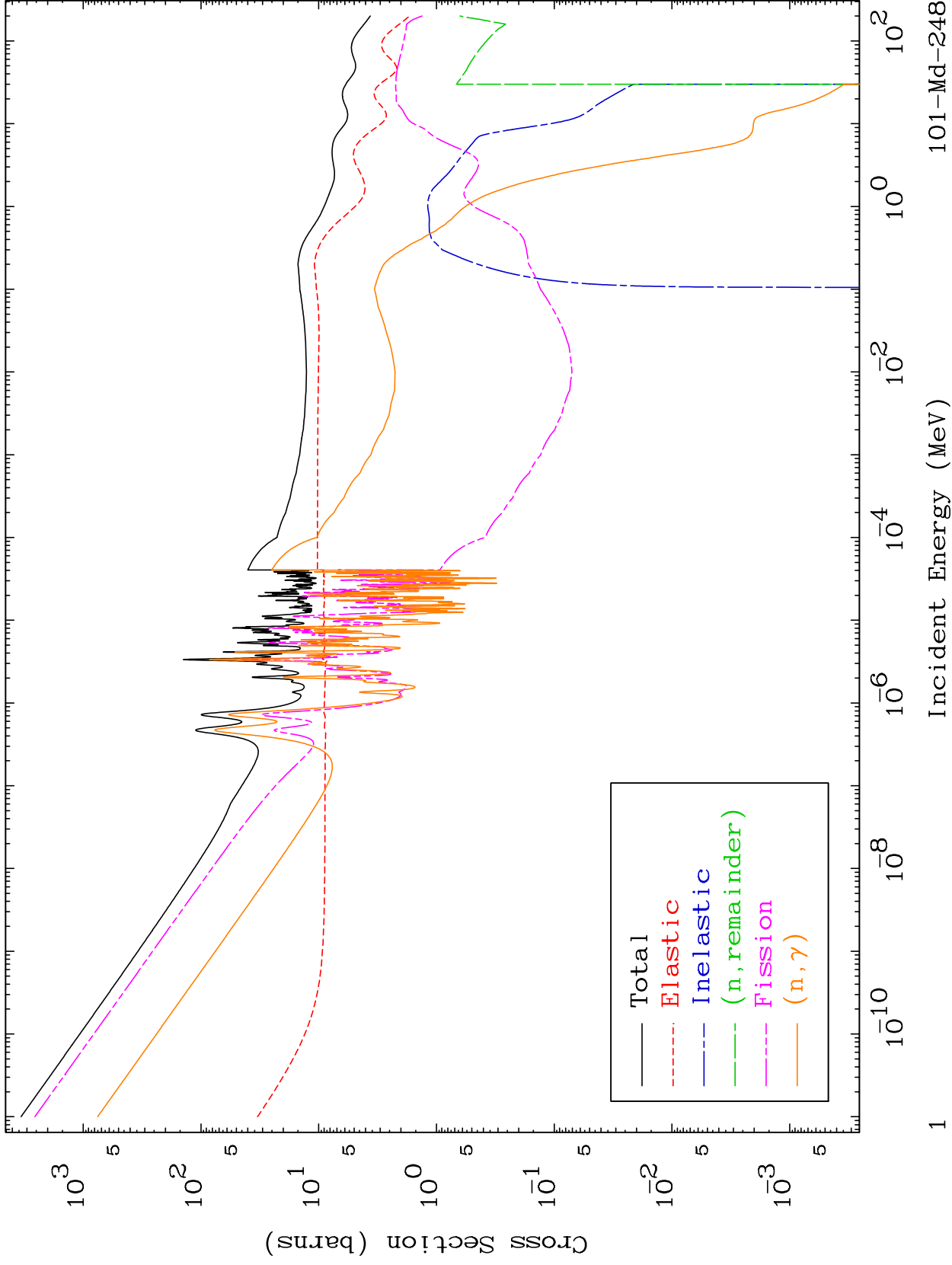
E.Mail:redcullen1@comcast.net
Web:home.comcast.net/~redcullen1

Press Mouse Button to Start

MAT 9949

Major
293 Kelvin Cross Sections

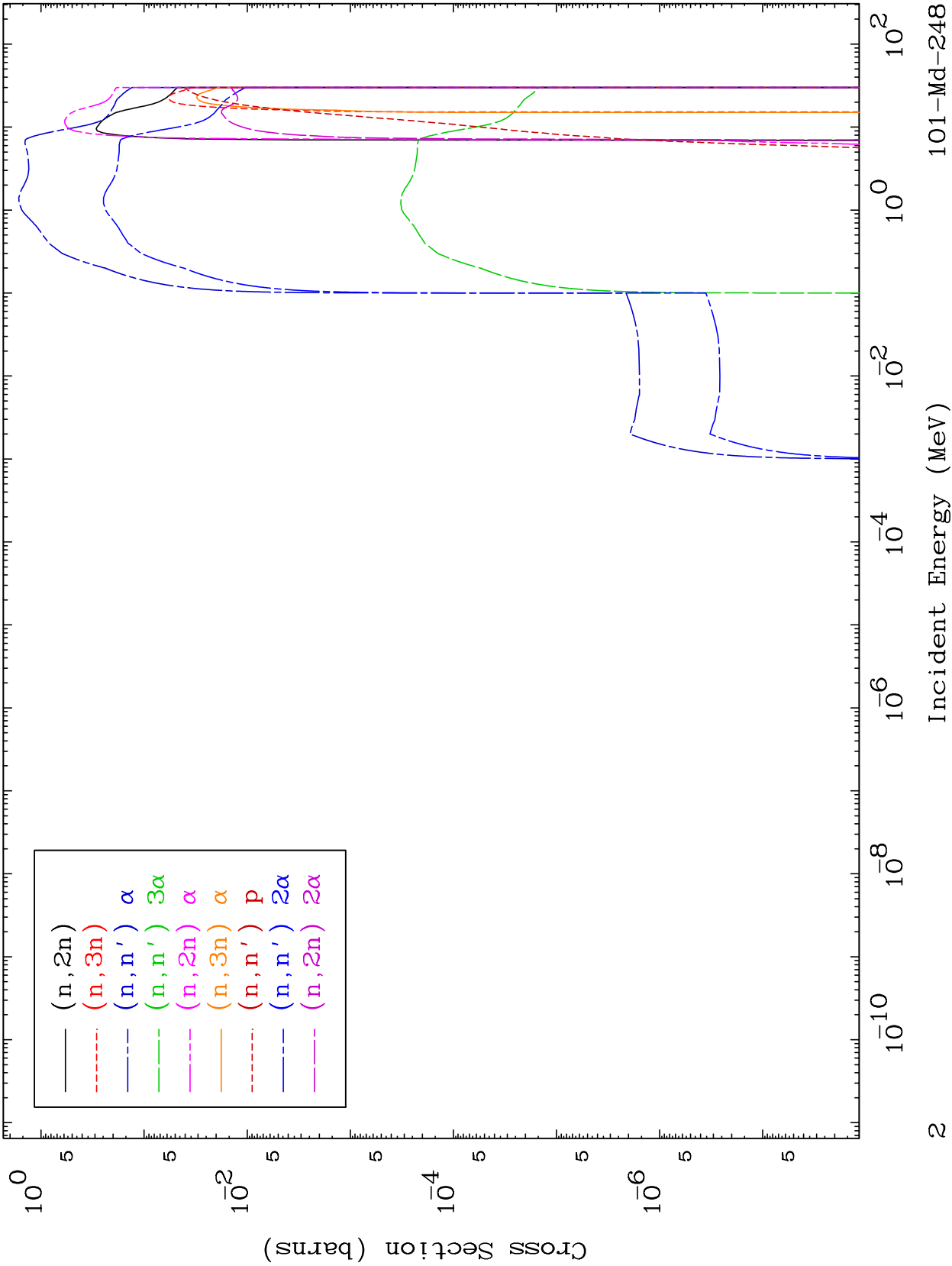
101-Md-248



MAT 9949

Neutron Production
293 Kelvin Cross Sections

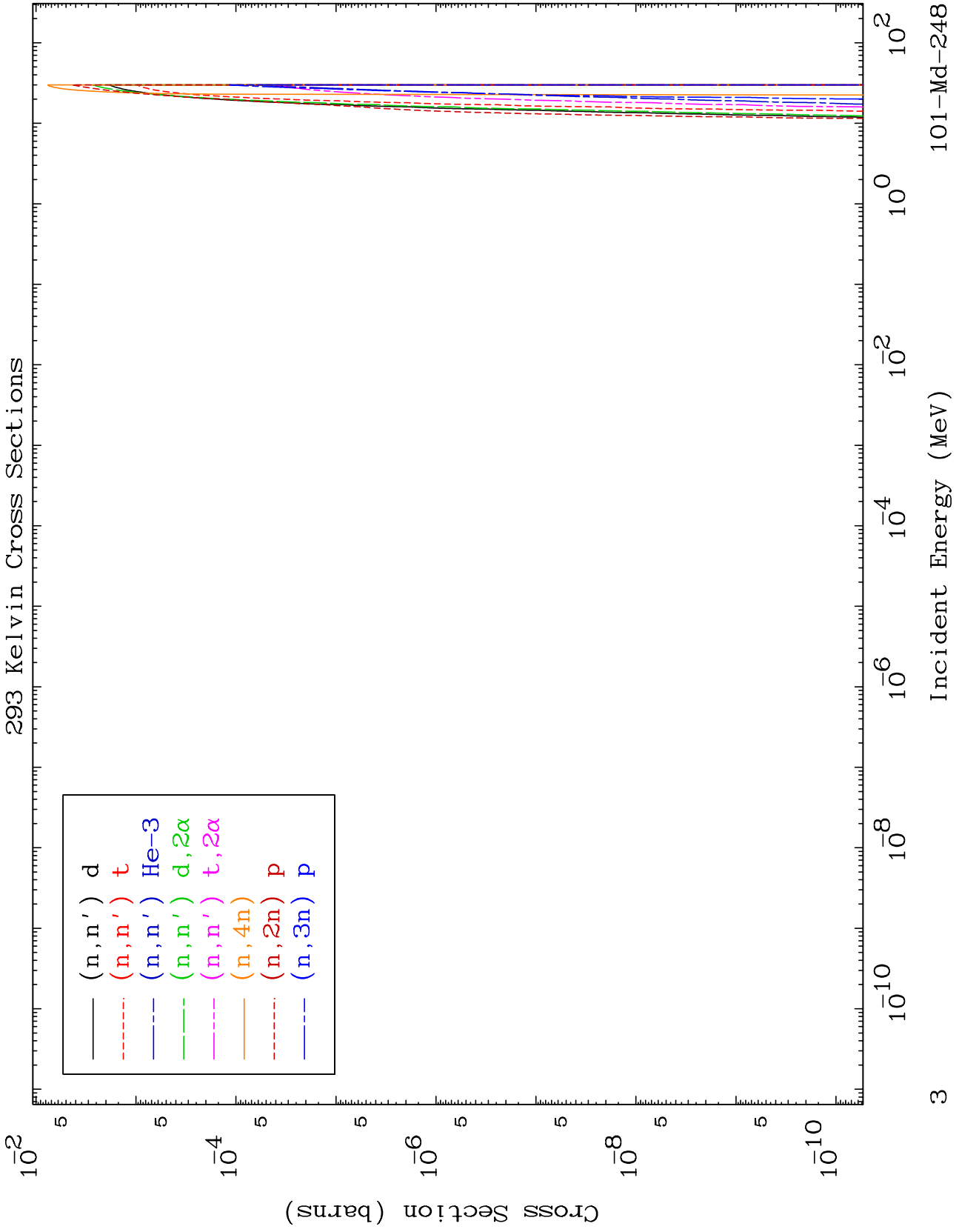
101-Md-248



MAT 9949

Neutron Production
293 Kelvin Cross Sections

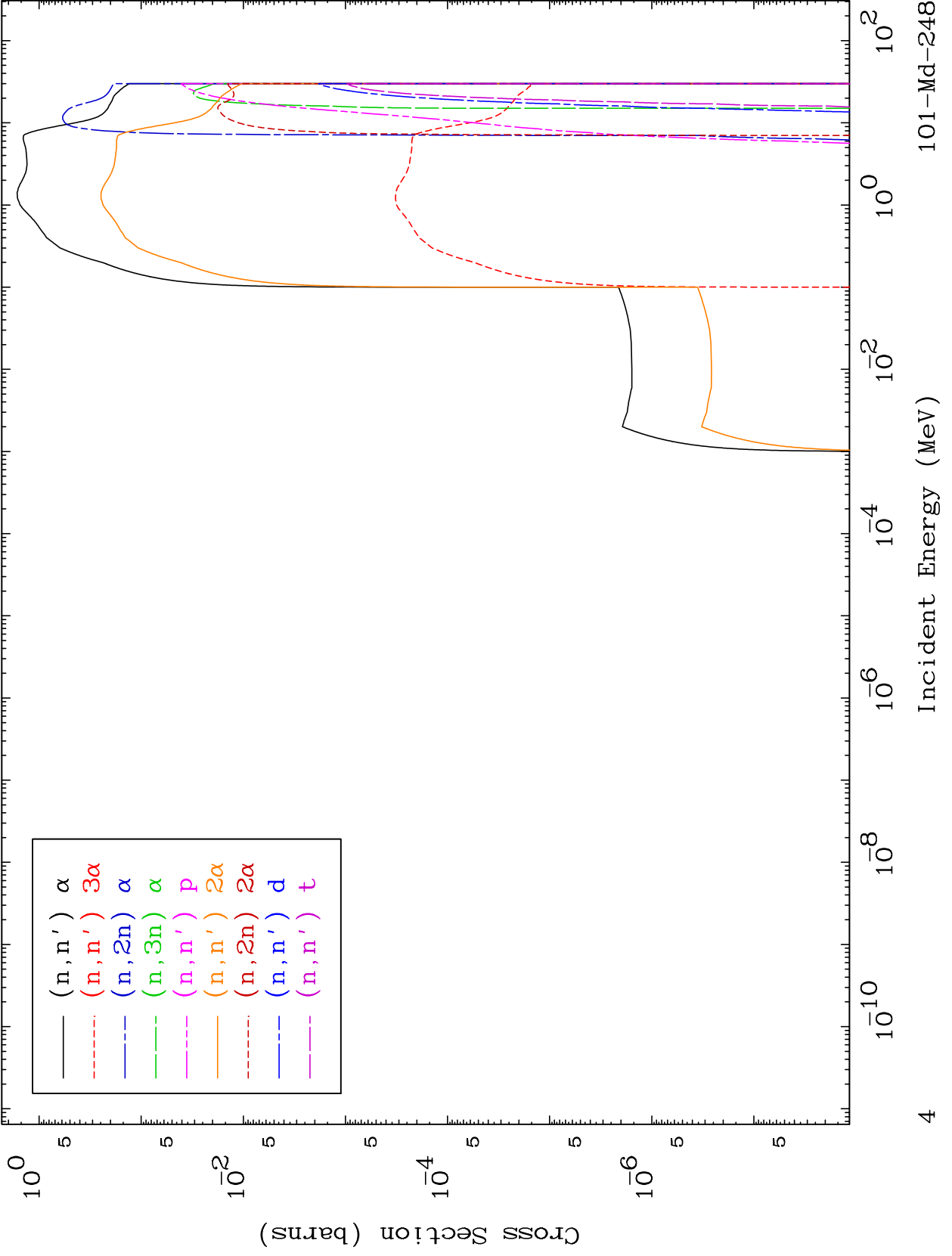
101-Md-248



MAT 9949

Charged Particle
293 Kelvin Cross Sections

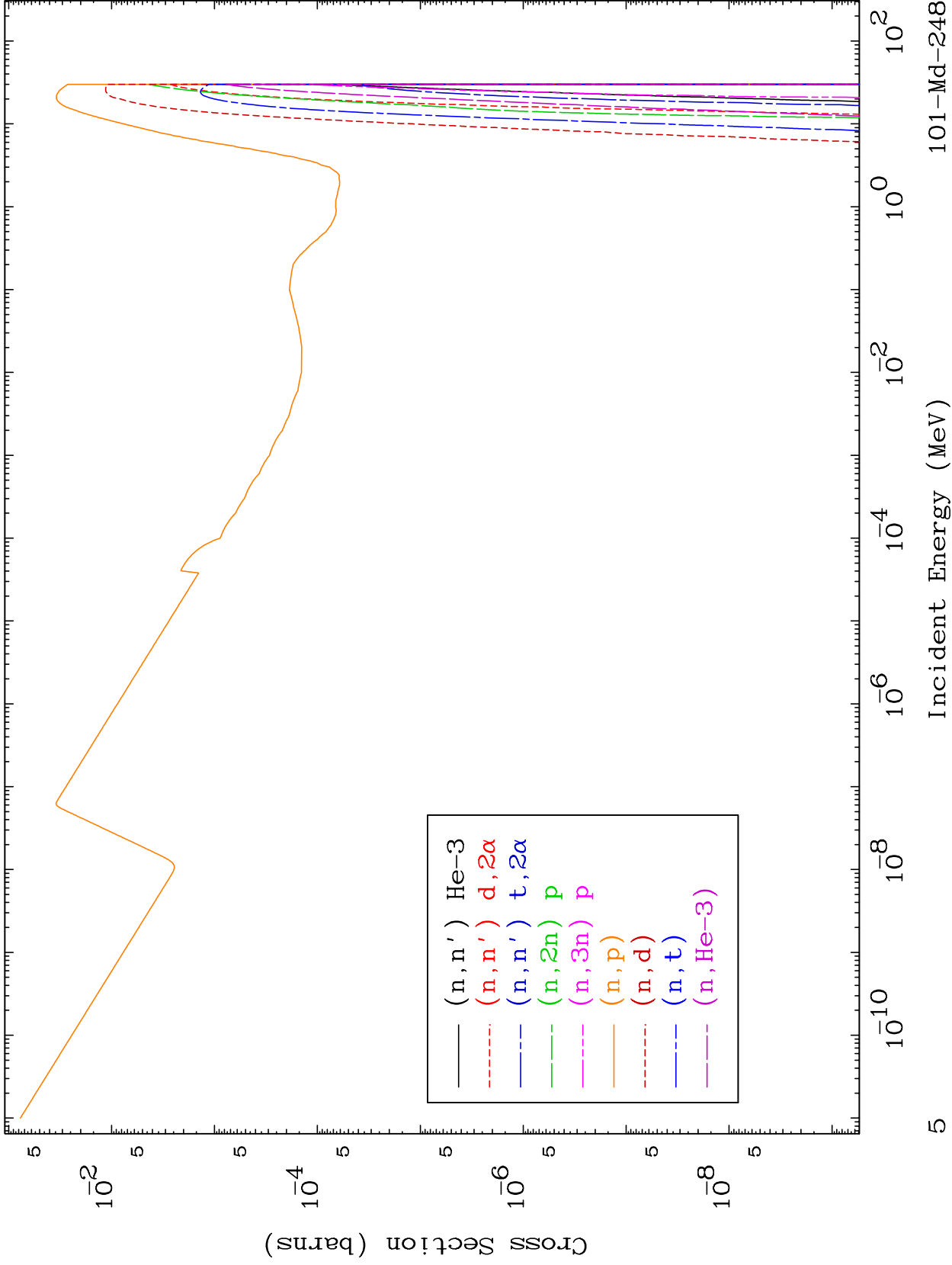
101-Md-248



MAT 9949

Charged Particle
293 Kelvin Cross Sections

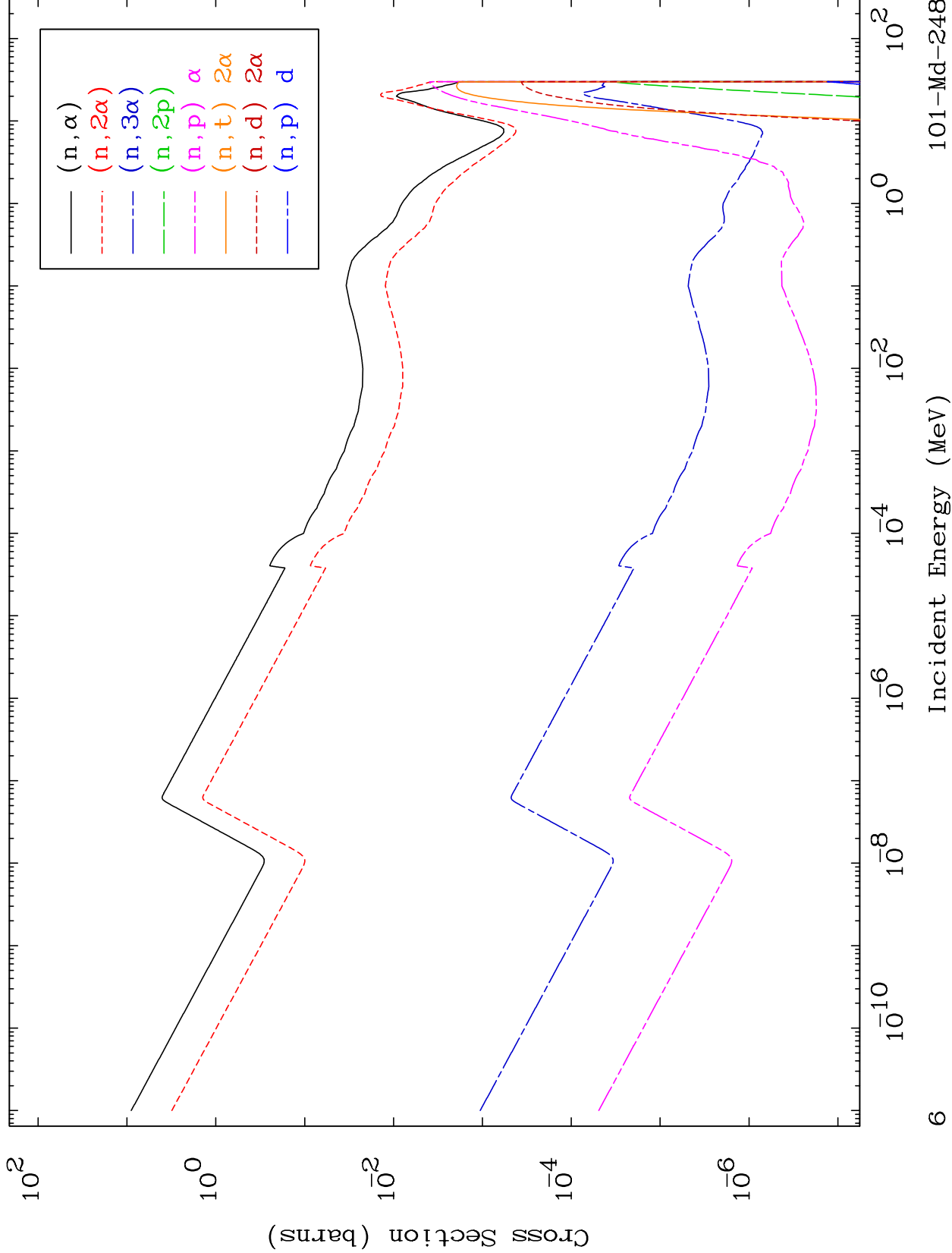
101-Md-248



MAT 9949

Charged Particle
293 Kelvin Cross Sections

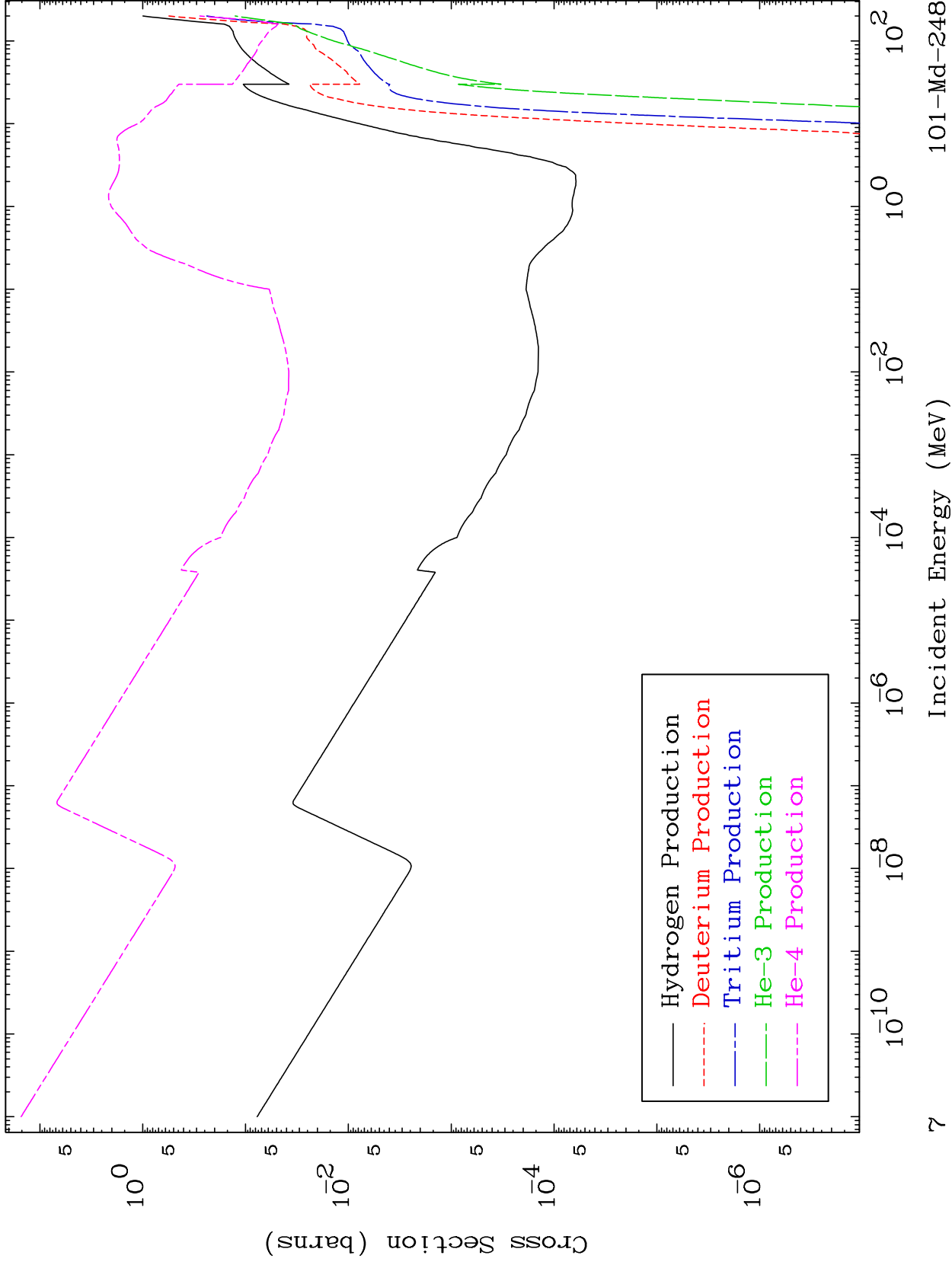
101-Md-248



MAT 9949

Particle Production
293 Kelvin Cross Sections

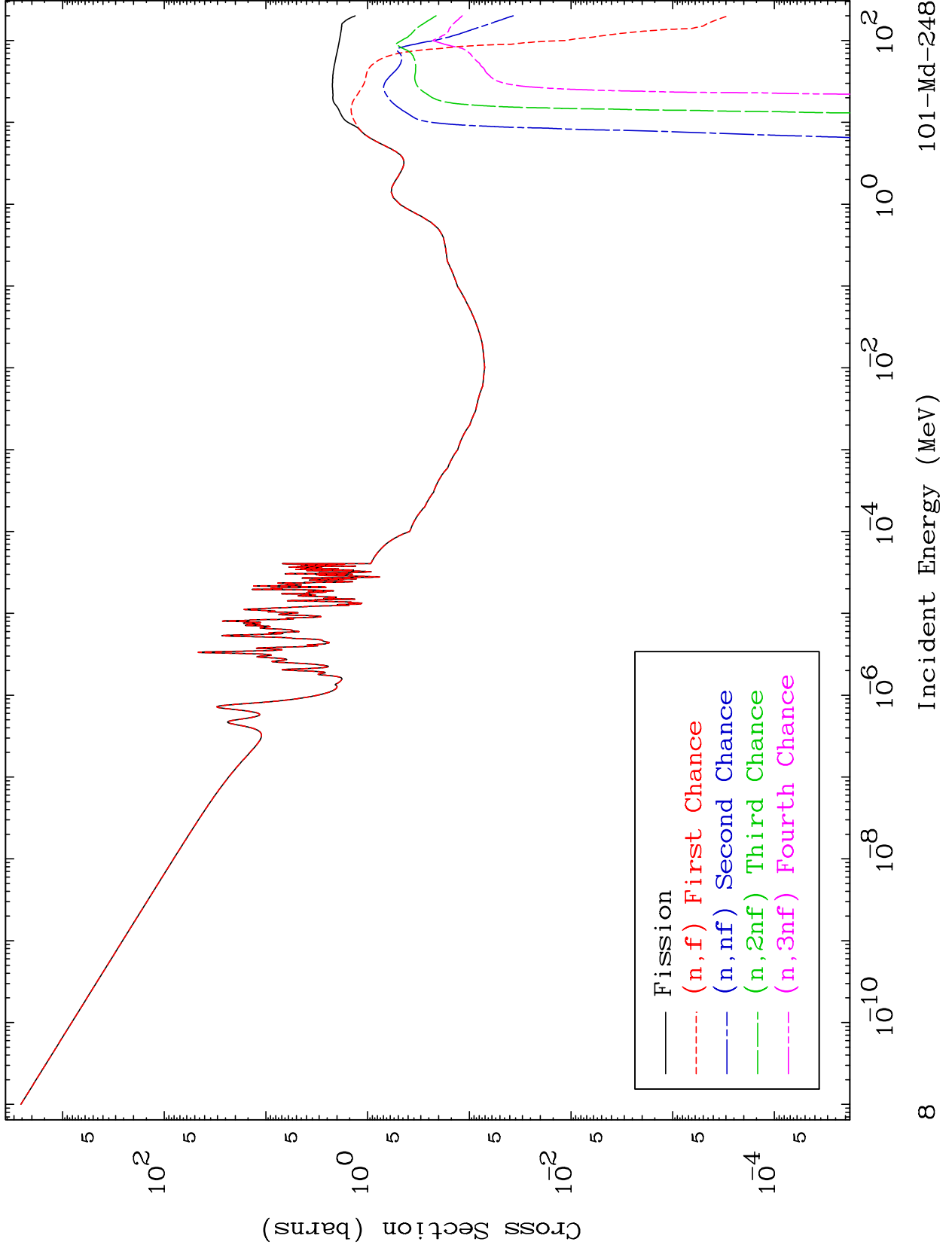
101-Md-248



MAT 9949

Fission
293 Kelvin Cross Sections

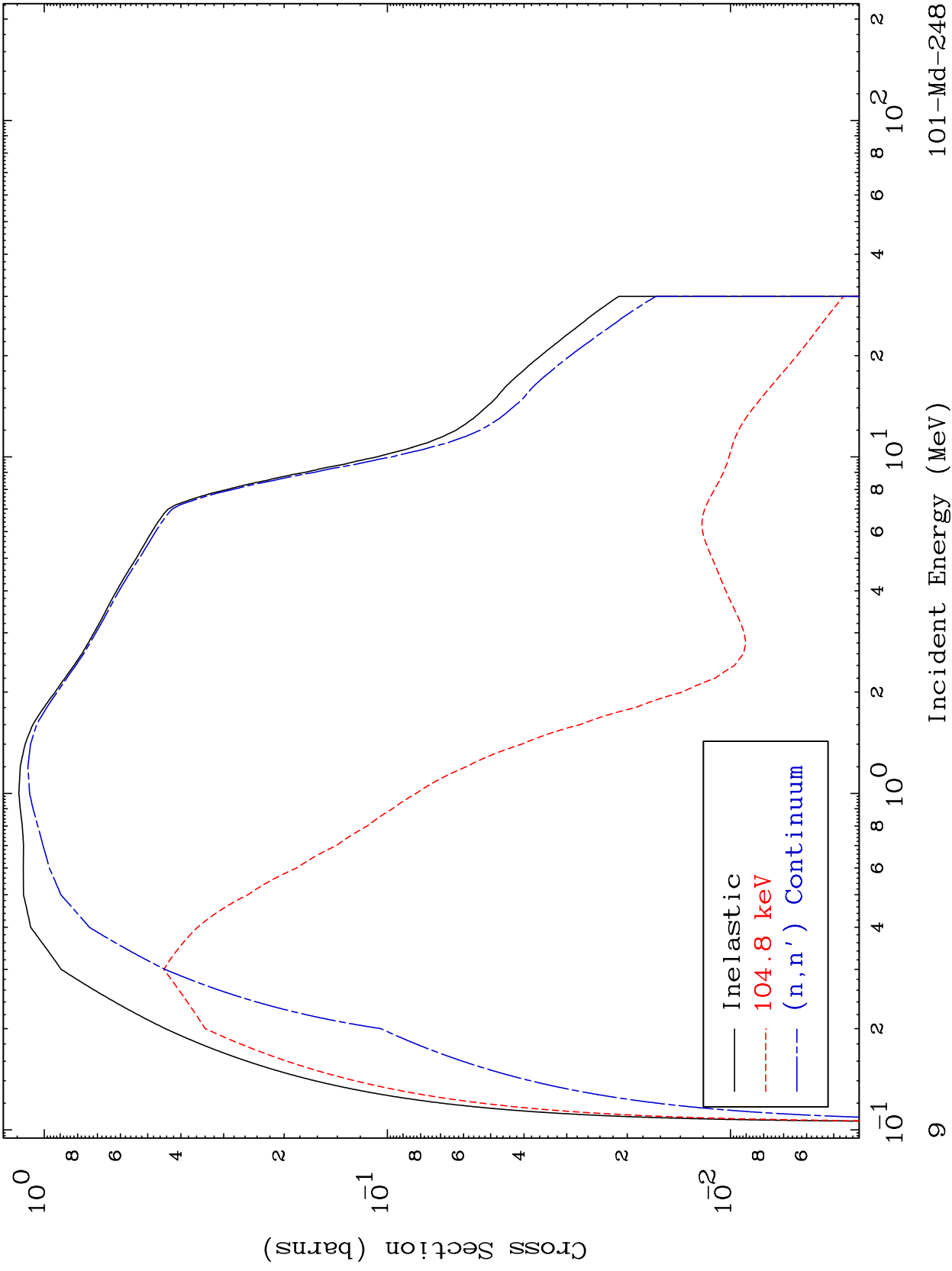
101-Md-248



MAT 9949

(n,n') Level
293 Kelvin Cross Sections

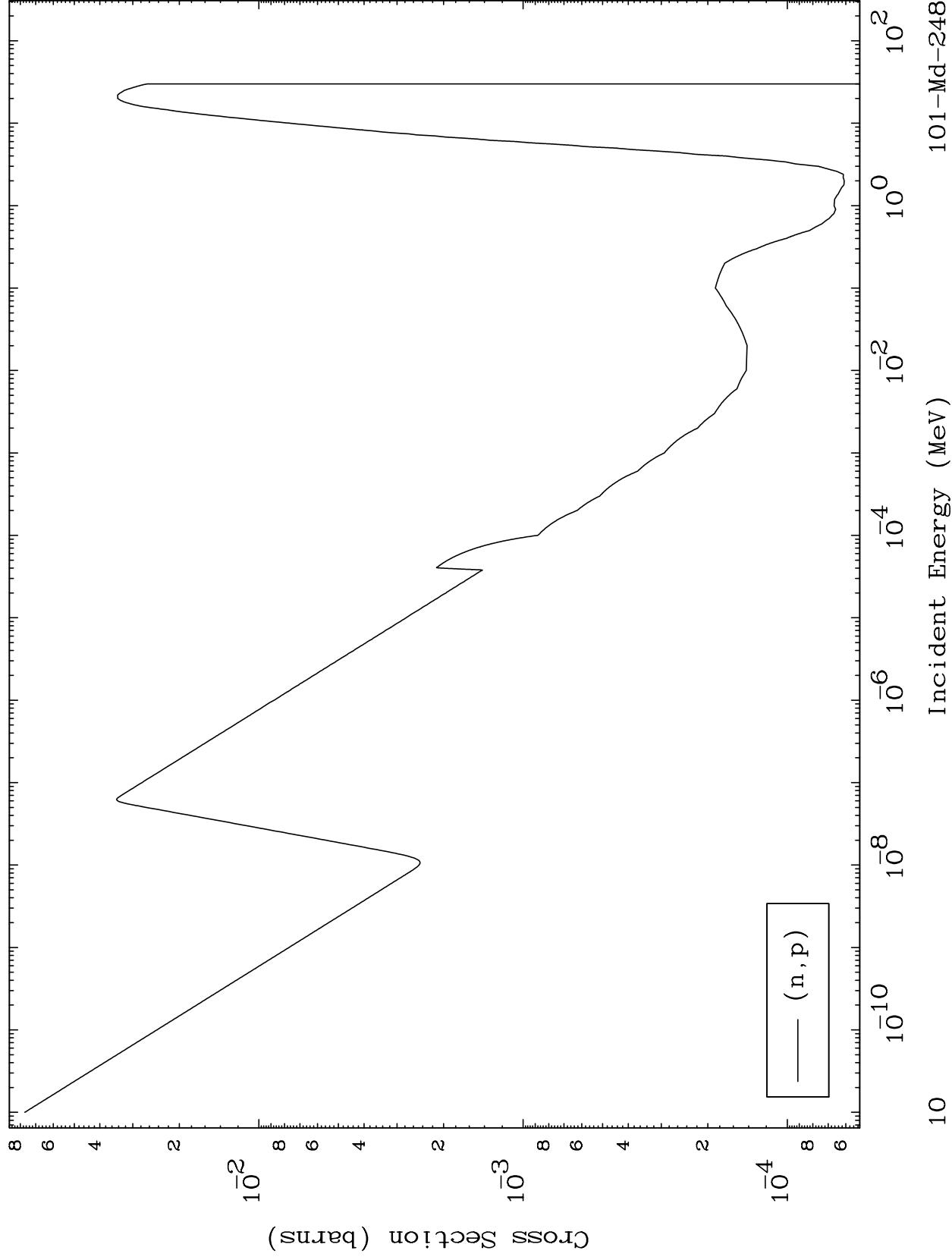
101-Md-248



MAT 9949

(n,p) Levels
293 Kelvin Cross Sections

101-Md-248



10

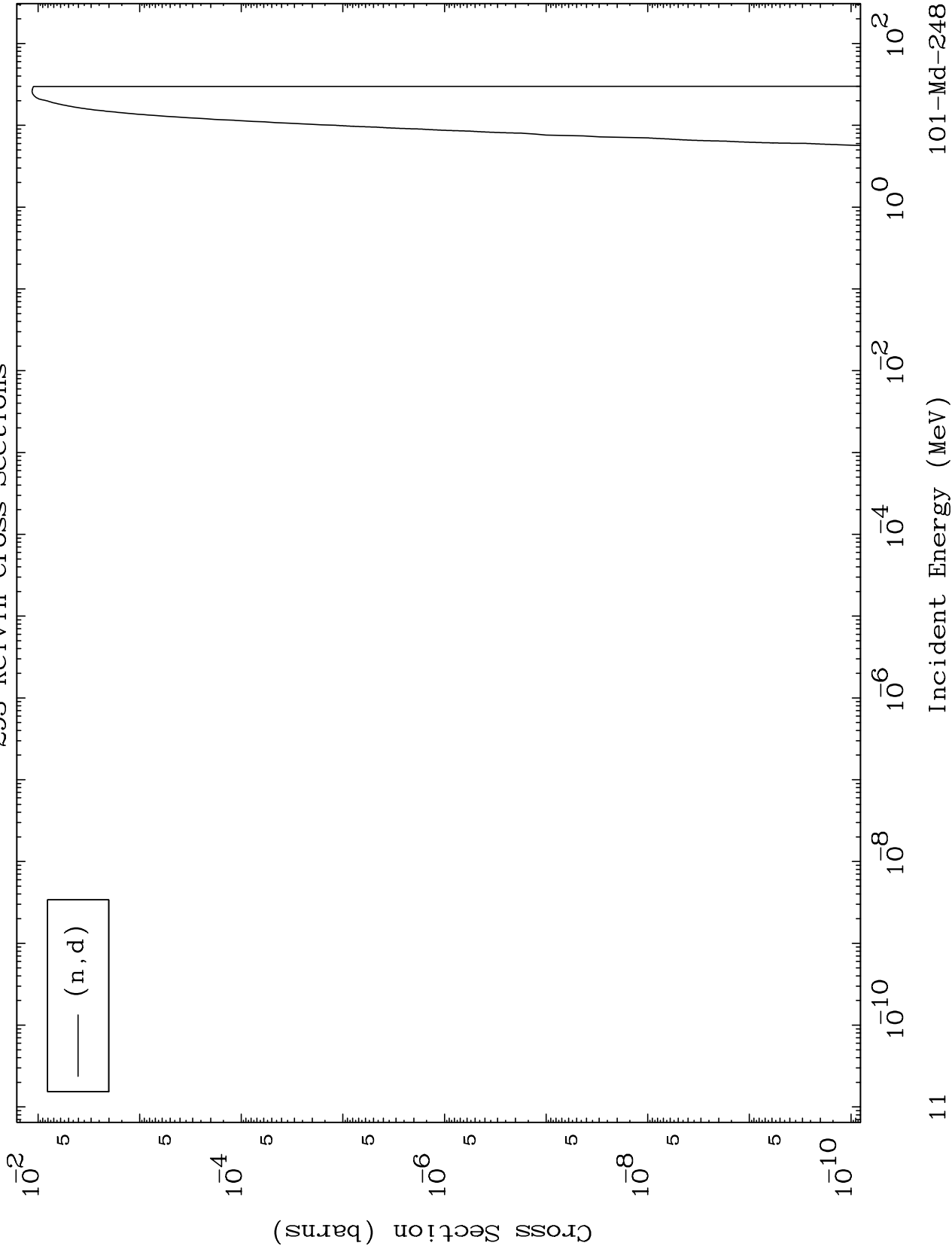
Incident Energy (MeV)

101-Md-248

MAT 9949

(n,d) Levels
293 Kelvin Cross Sections

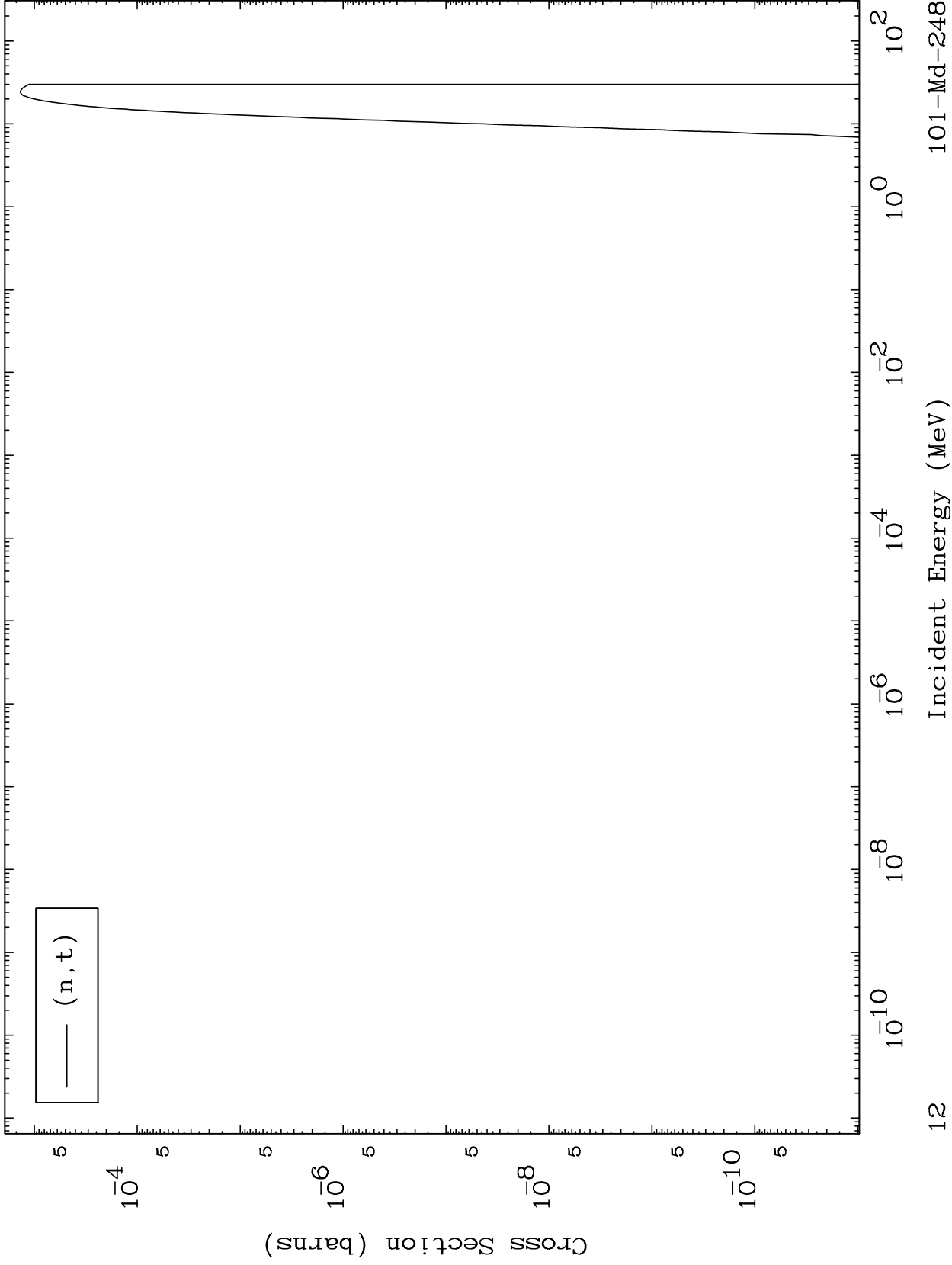
101-Md-248



MAT 9949

(n,t) Levels
293 Kelvin Cross Sections

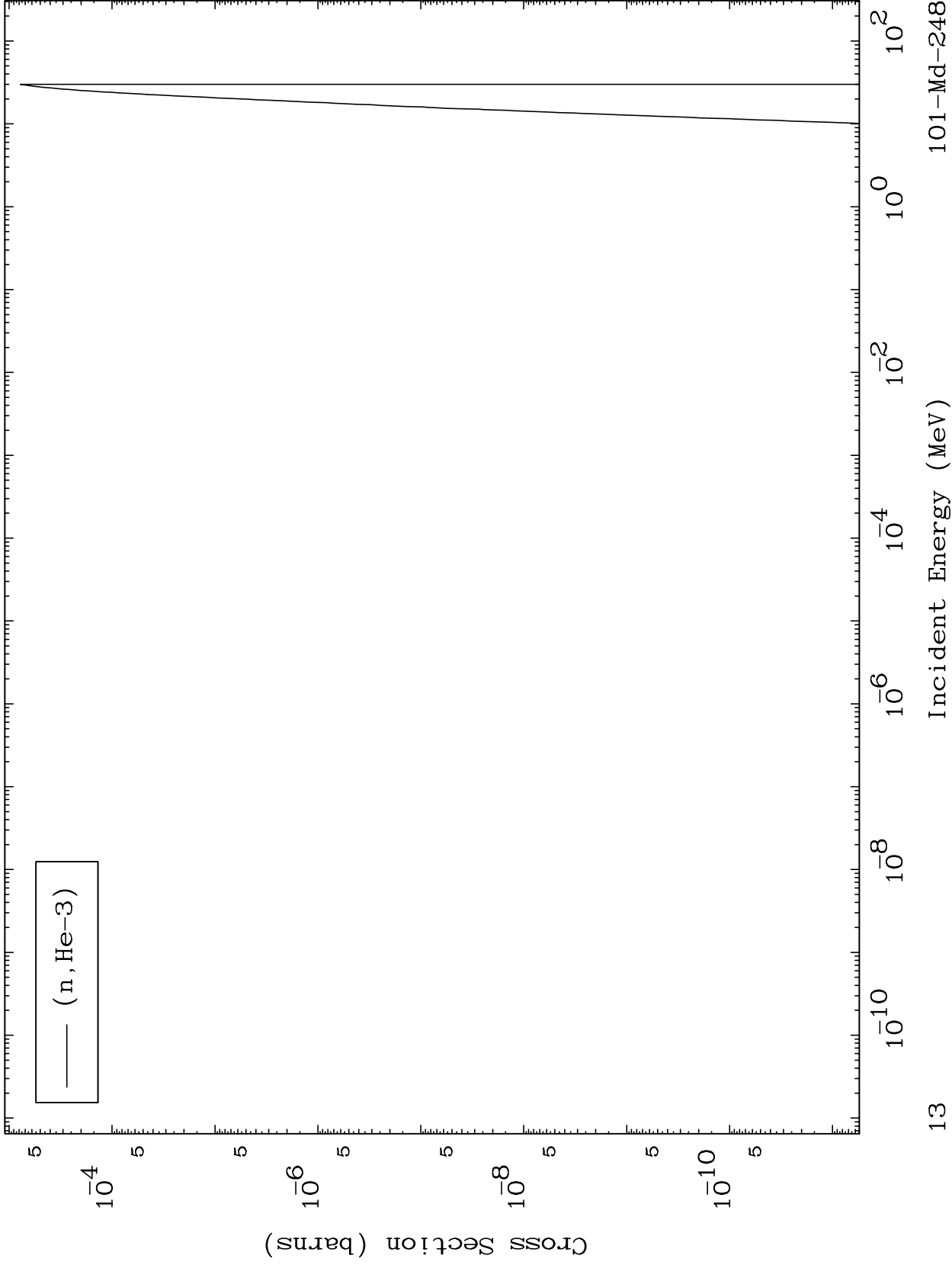
101-Md-248

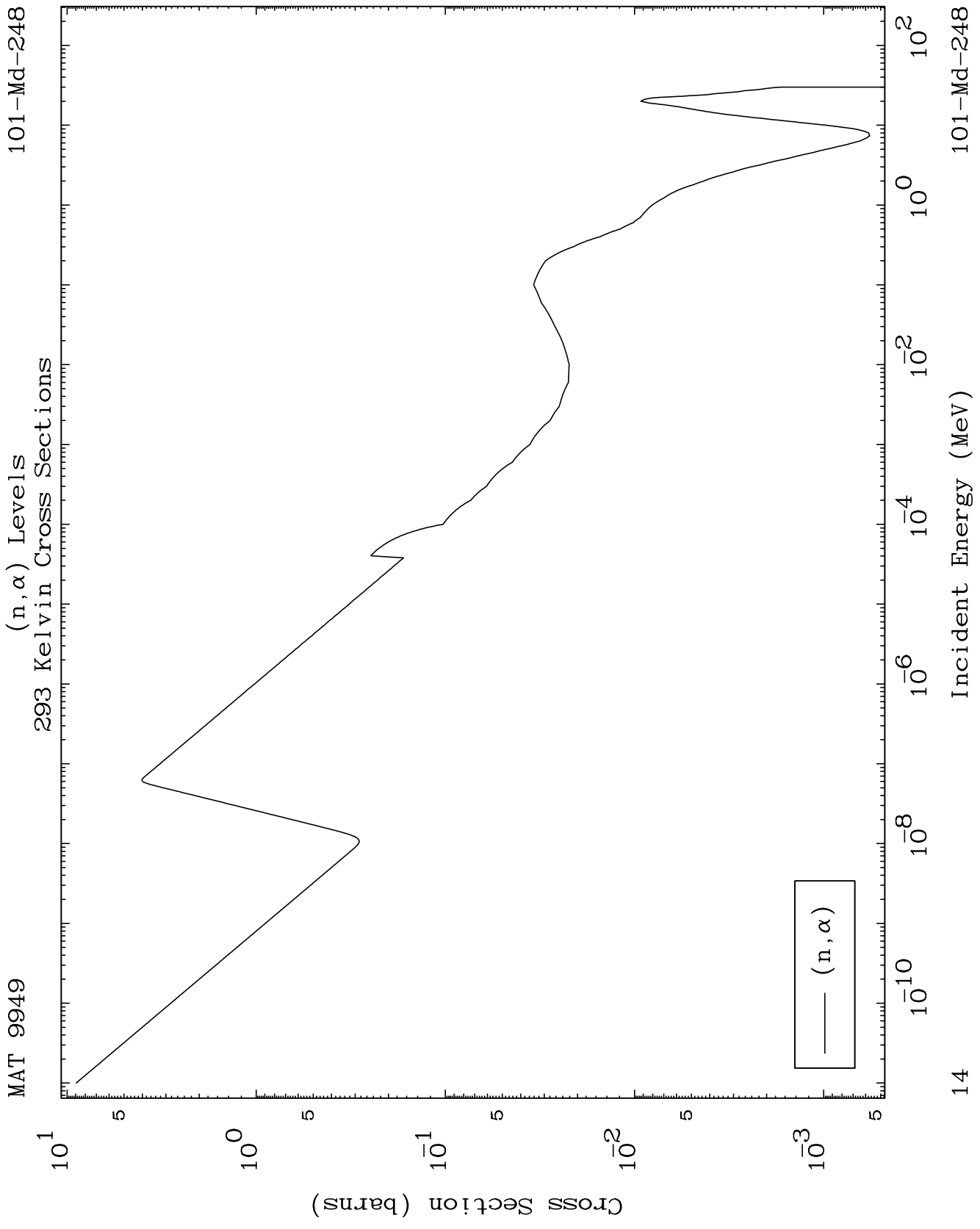


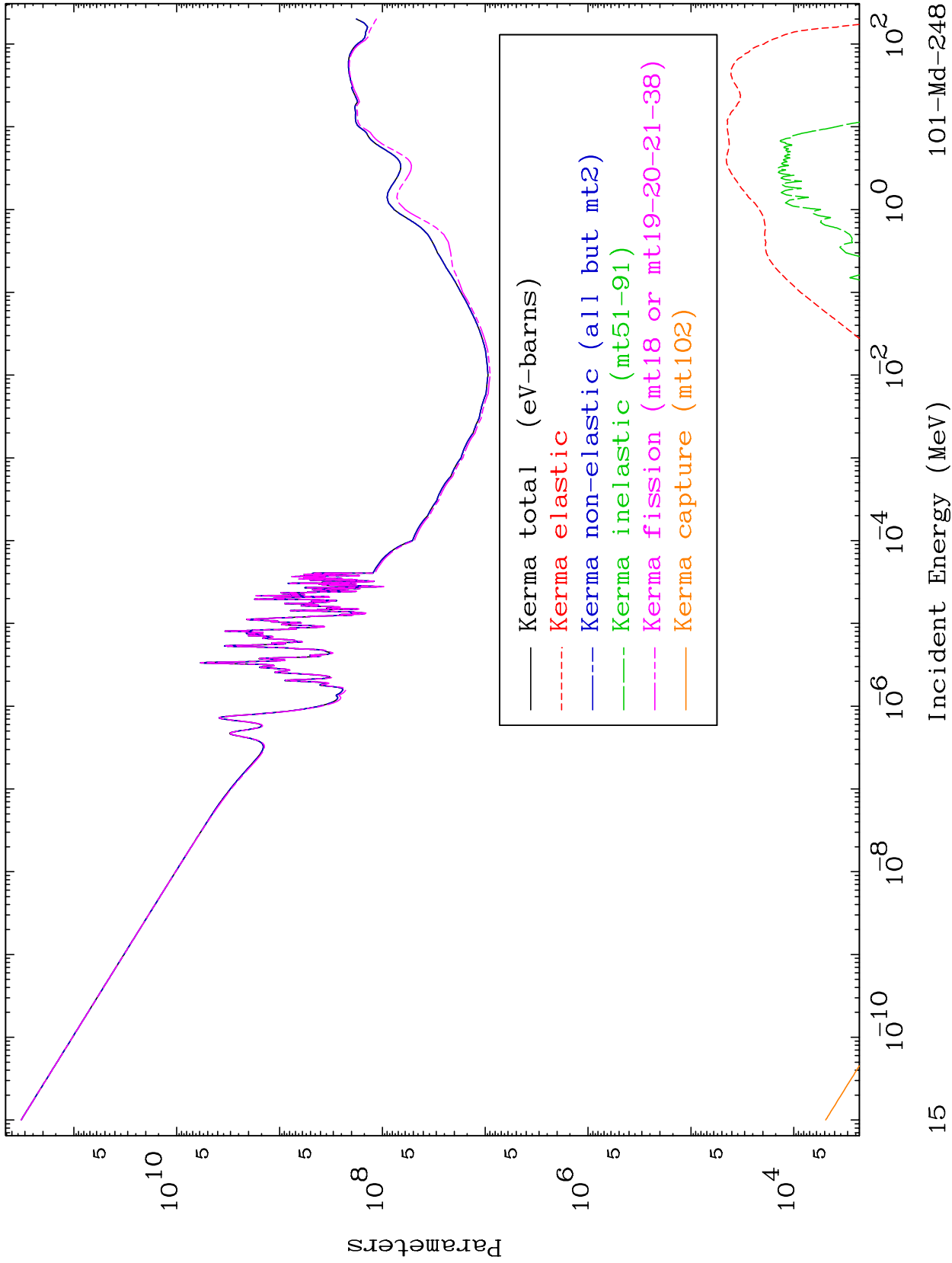
MAT 9949

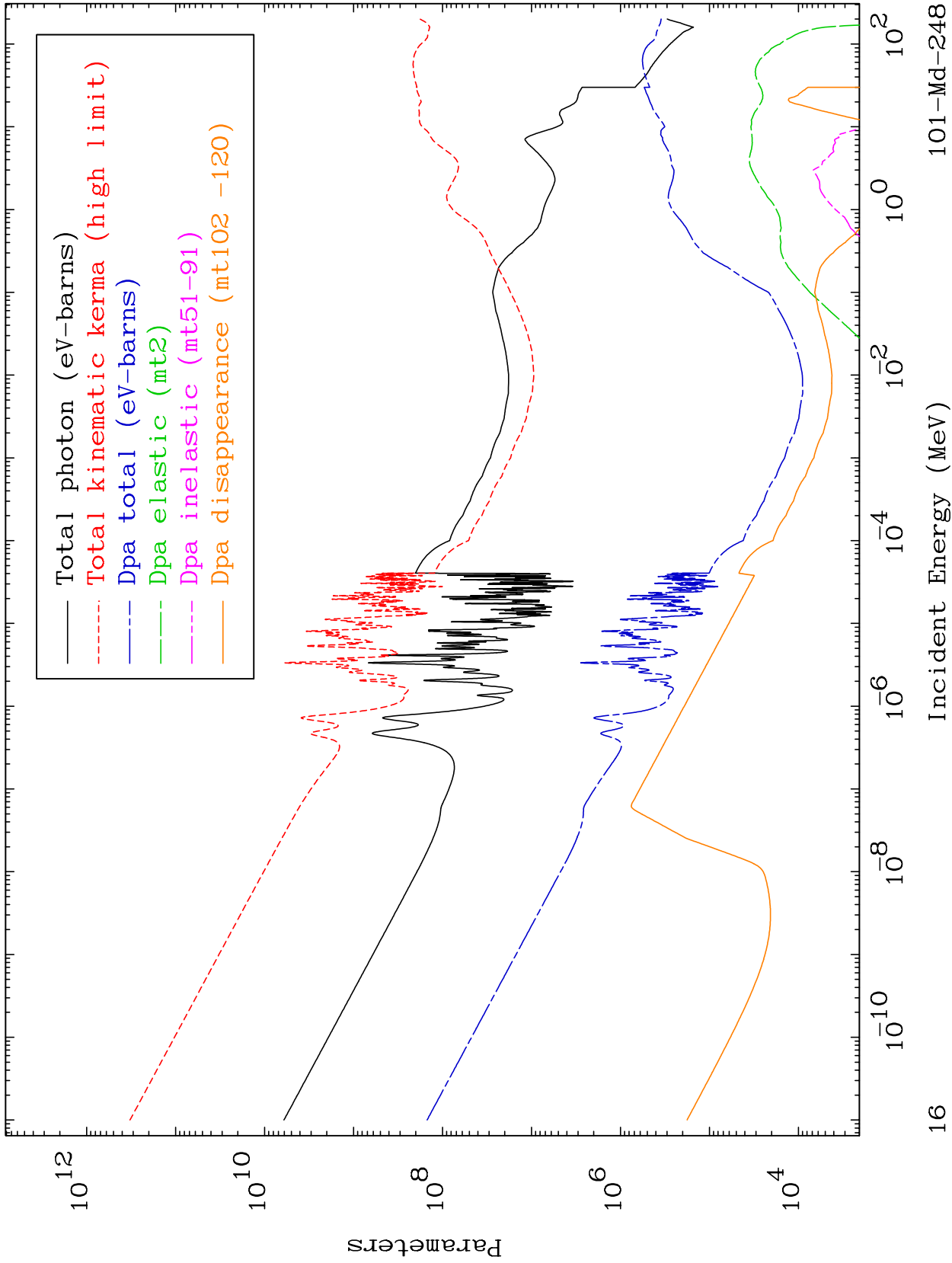
(n,He3) Levels
293 Kelvin Cross Sections

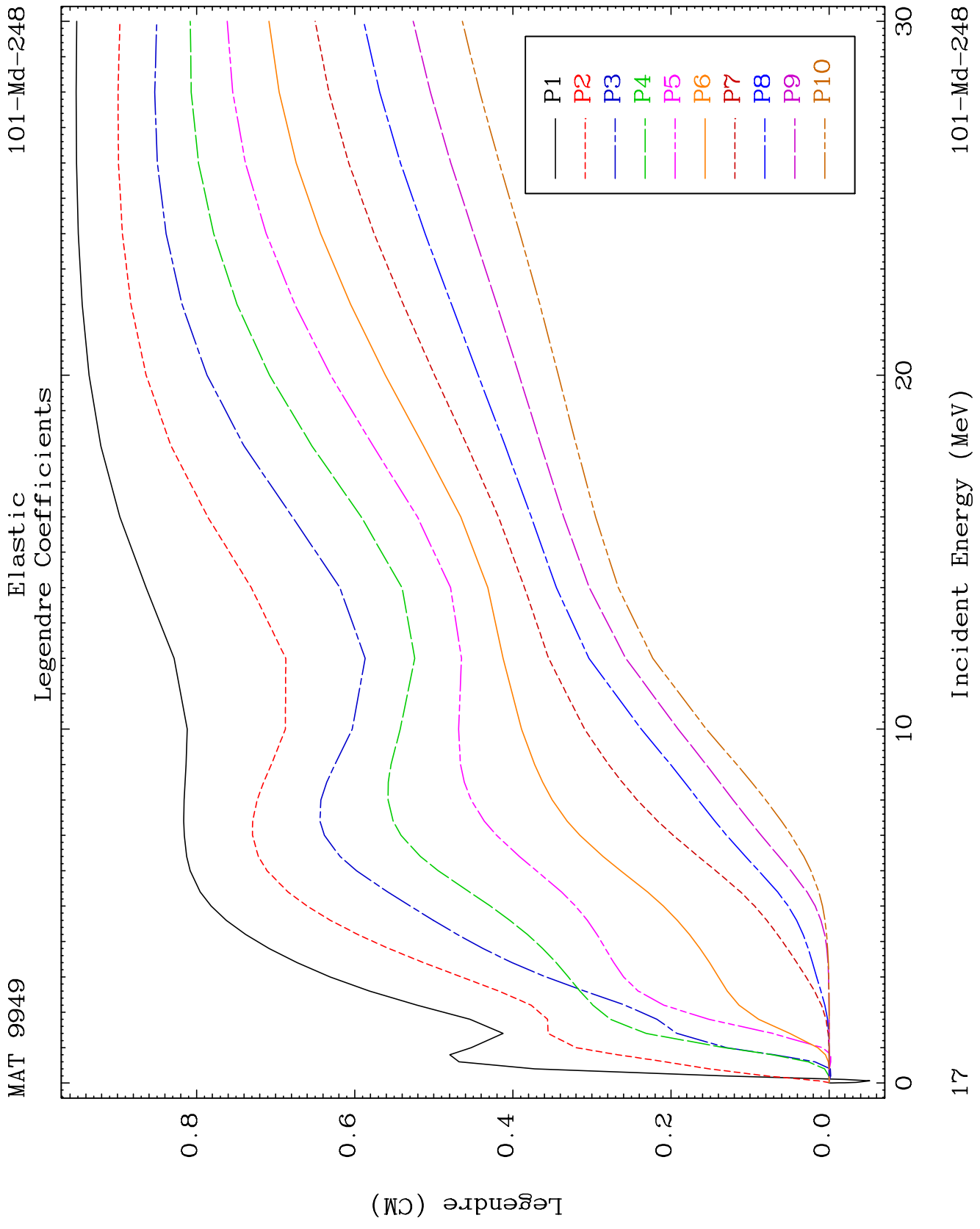
101-Md-248







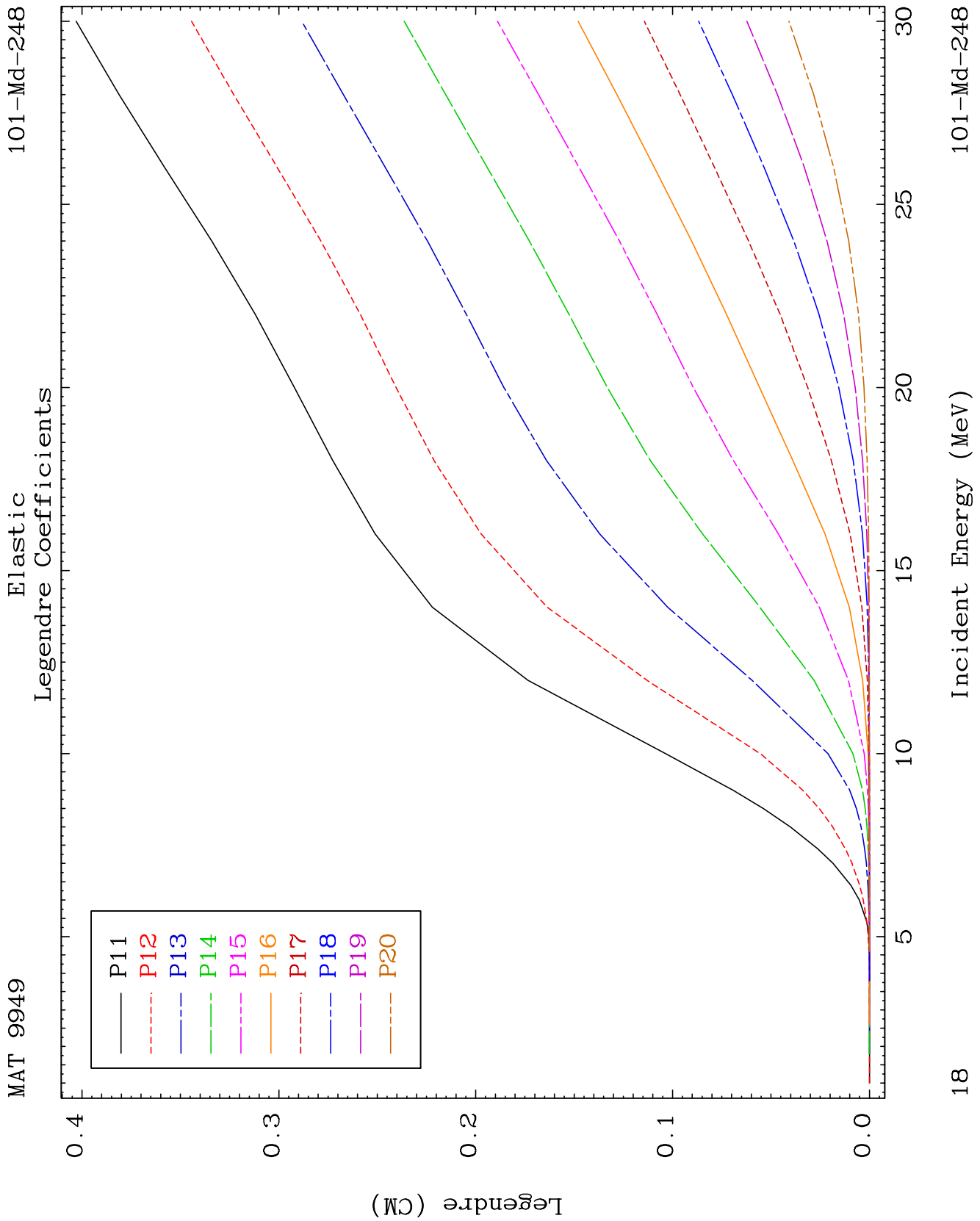




101-Md-248

Incident Energy (MeV)

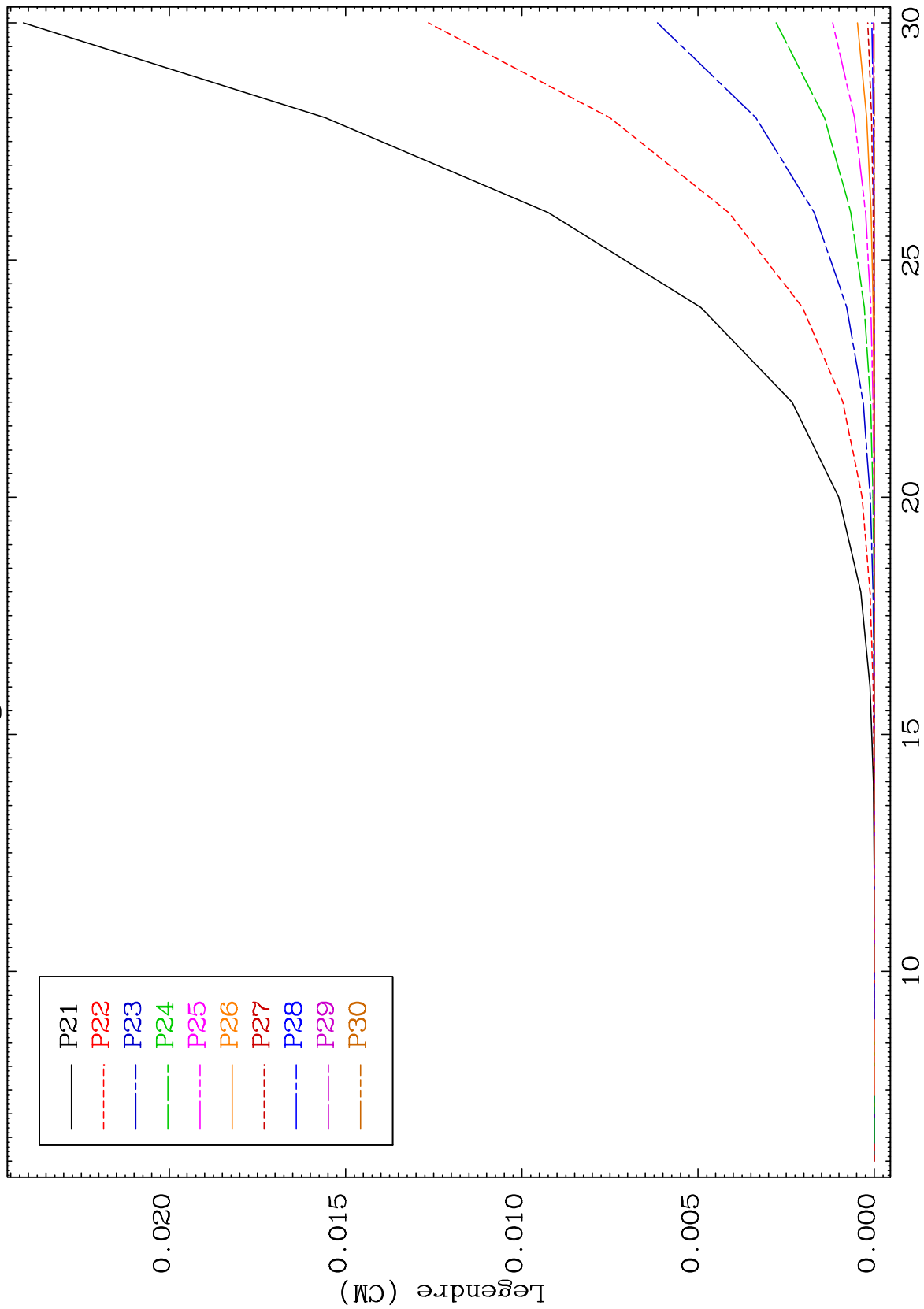
17



MAT 9949

Elastic Legendre Coefficients

101-Md-248



19

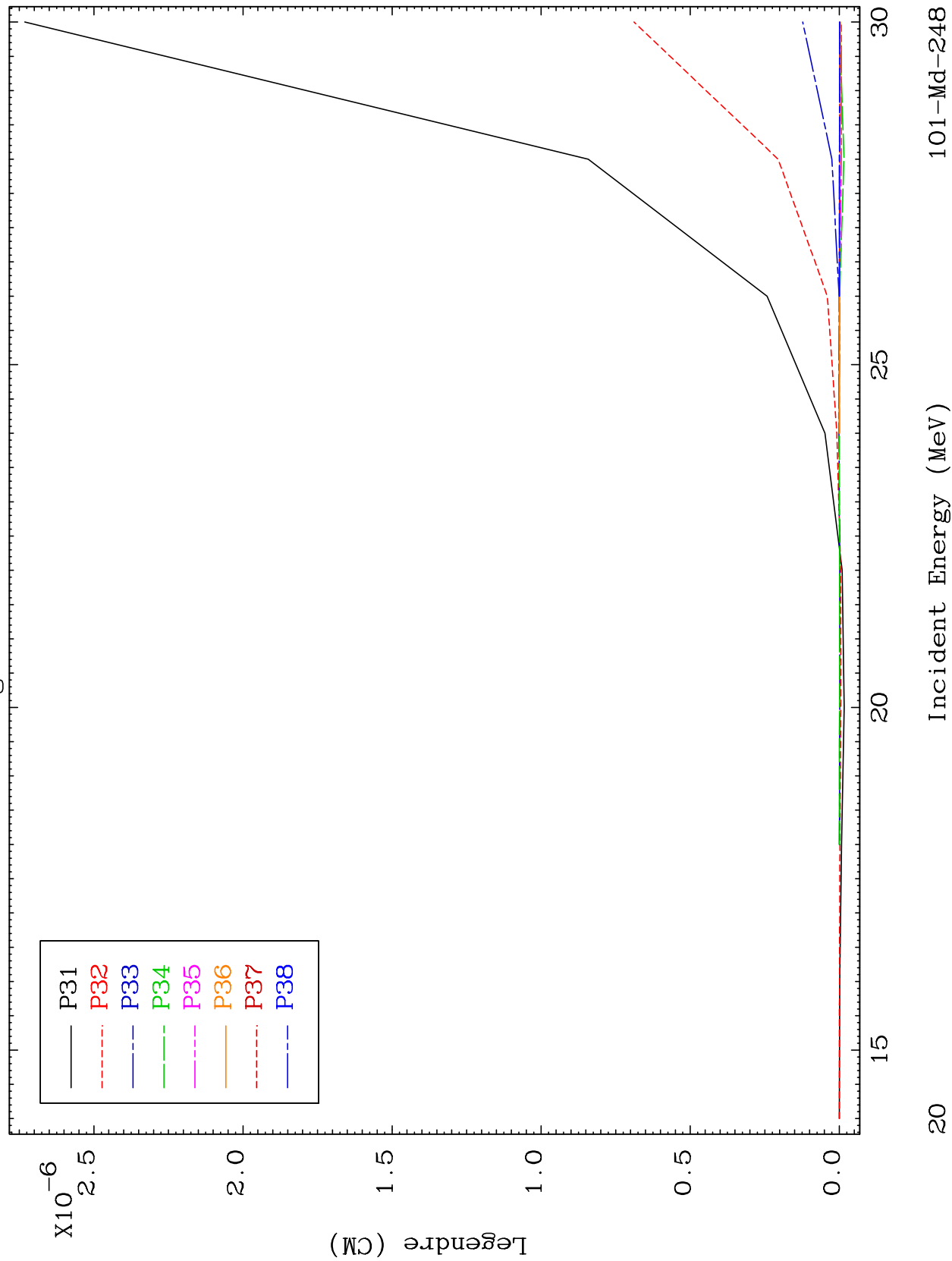
Incident Energy (MeV)

101-Md-248

MAT 9949

Elastic Legendre Coefficients

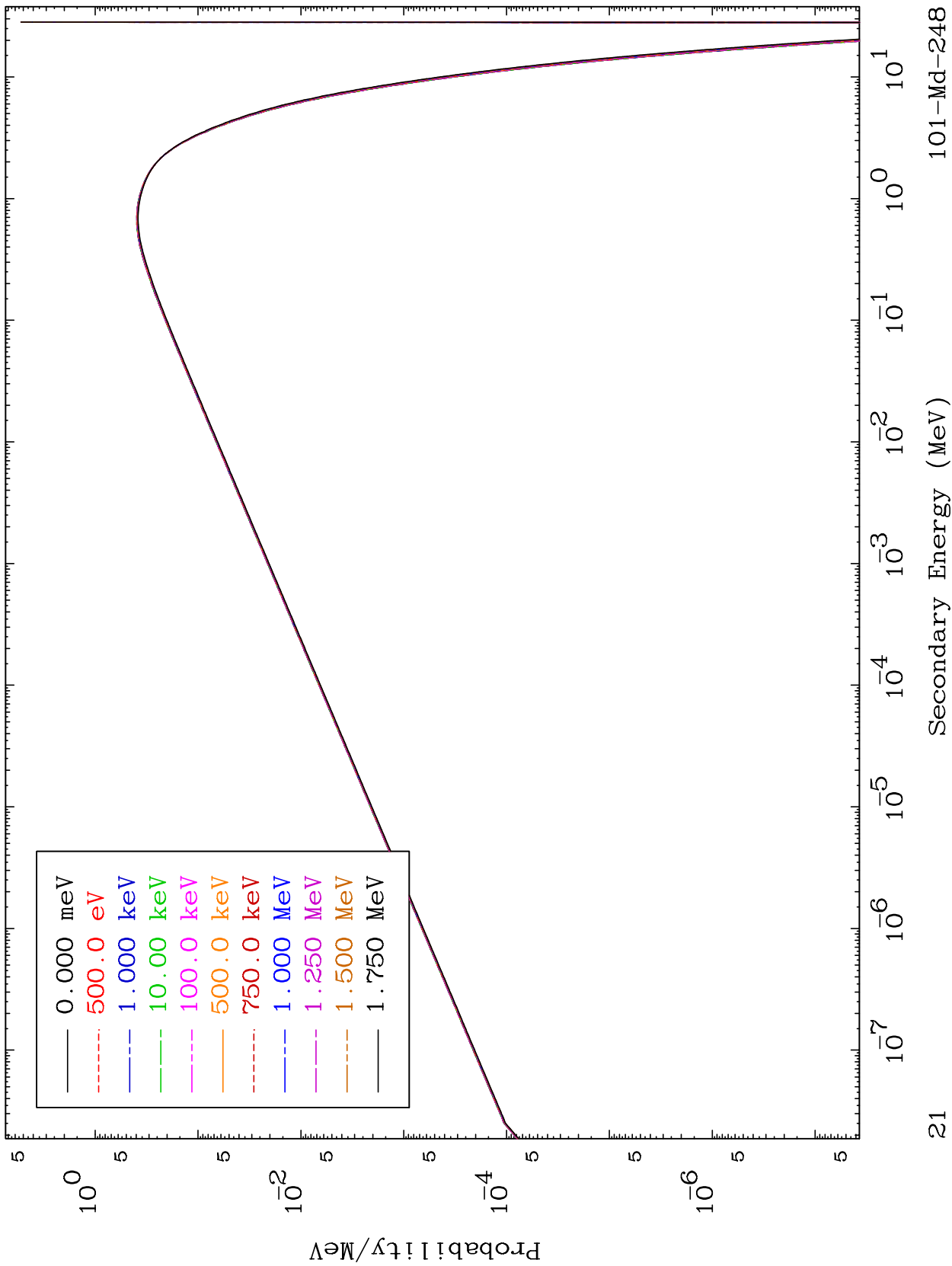
101-Md-248



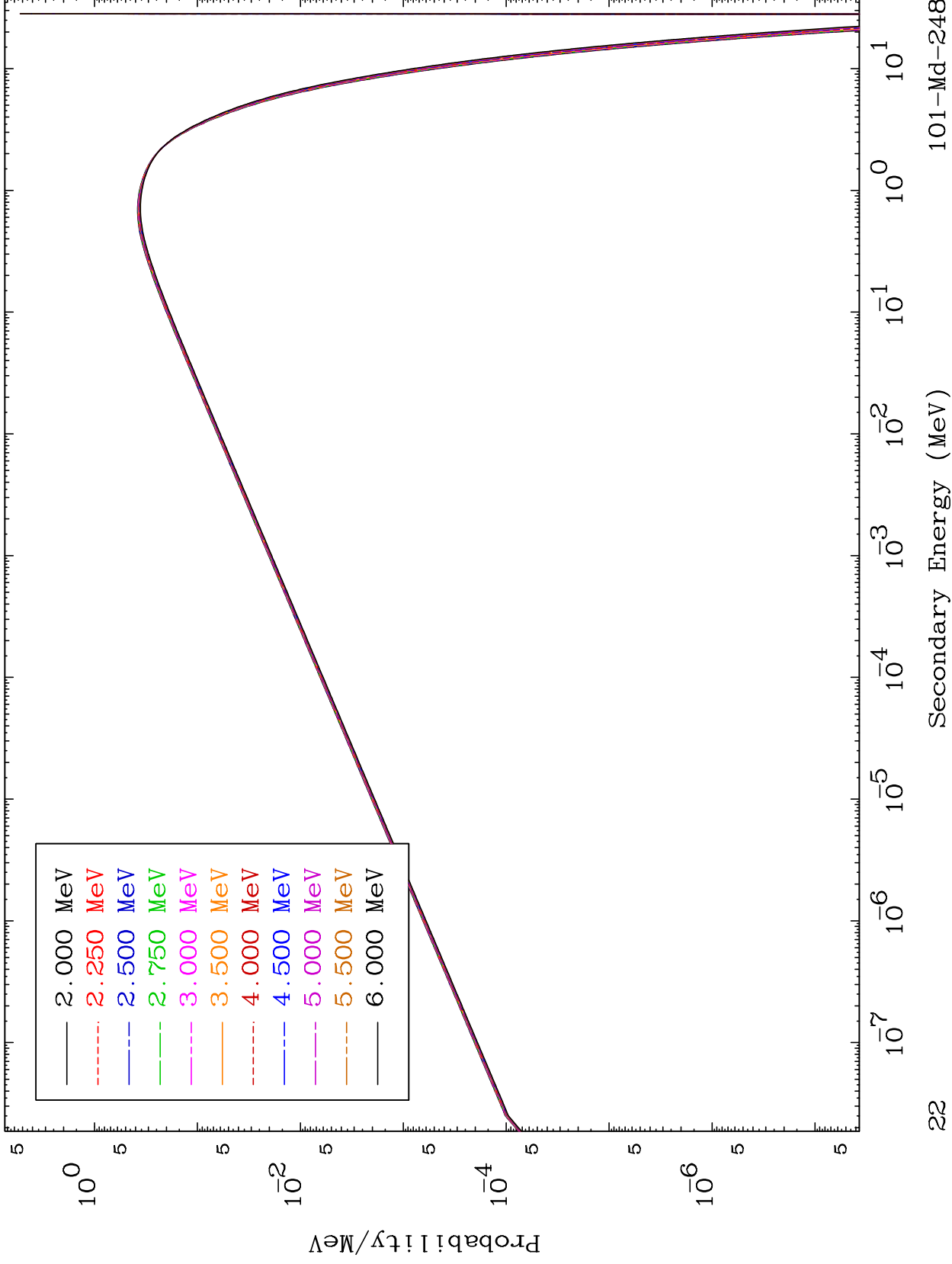
20

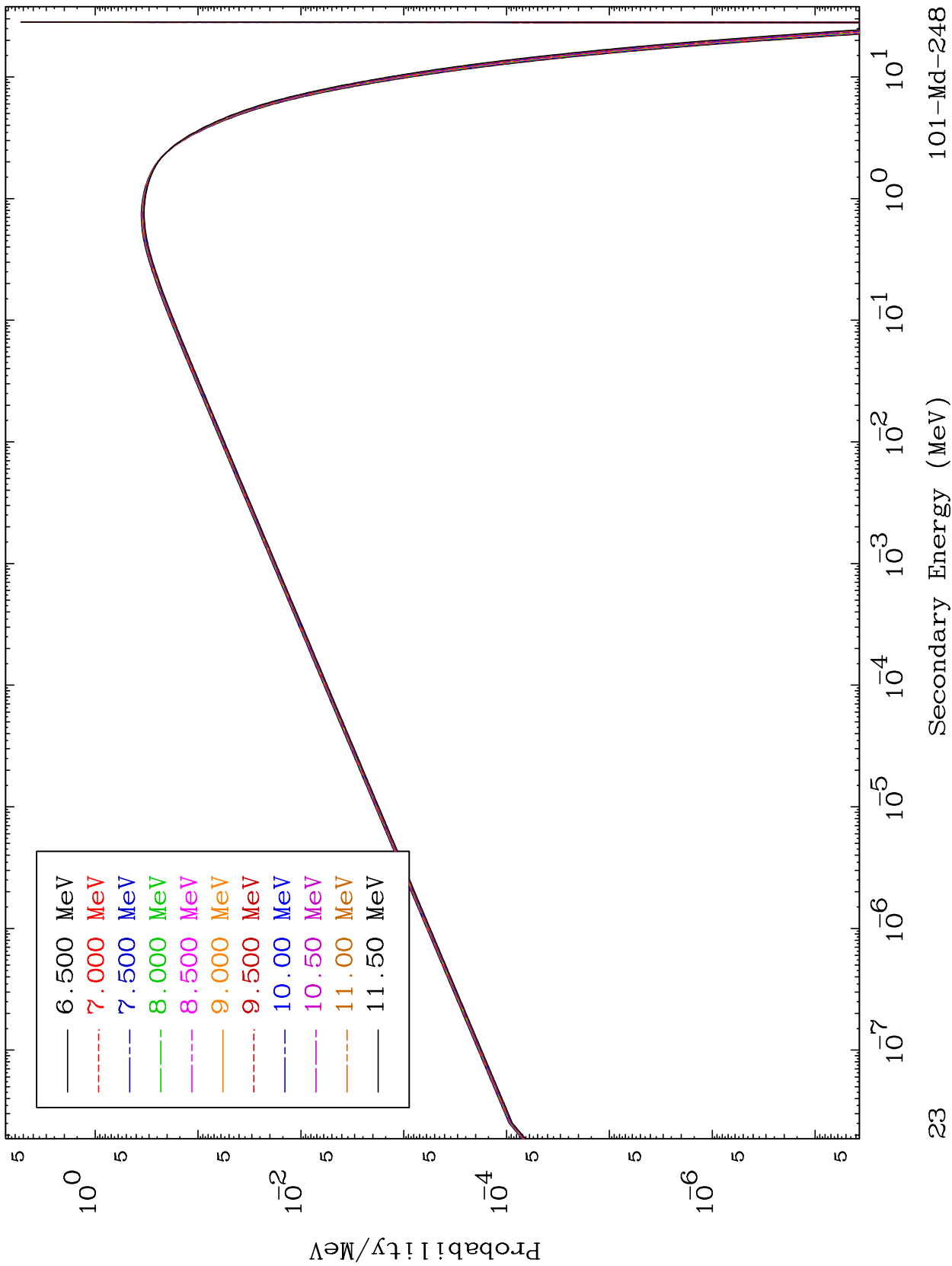
Incident Energy (MeV)

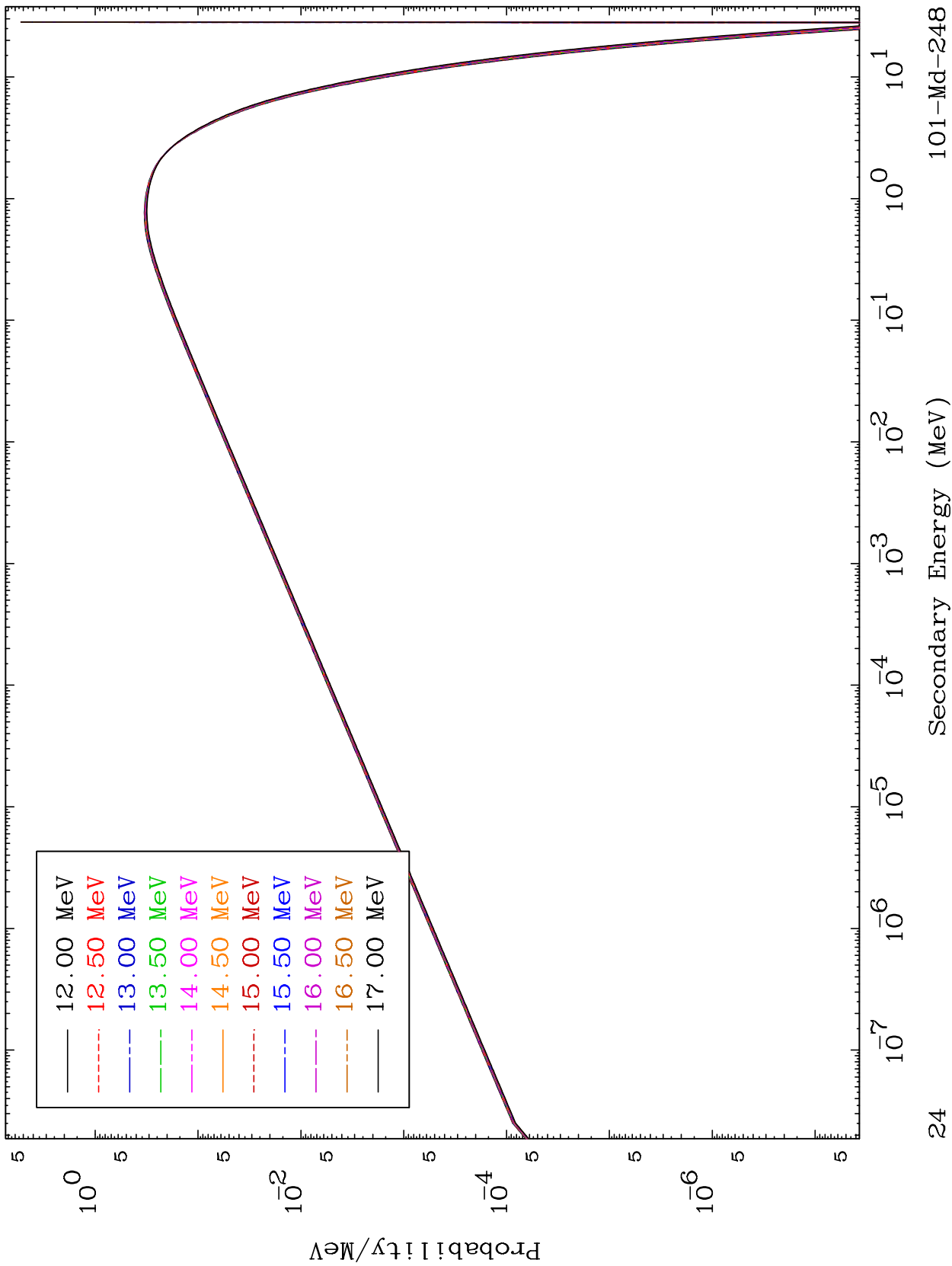
101-Md-248



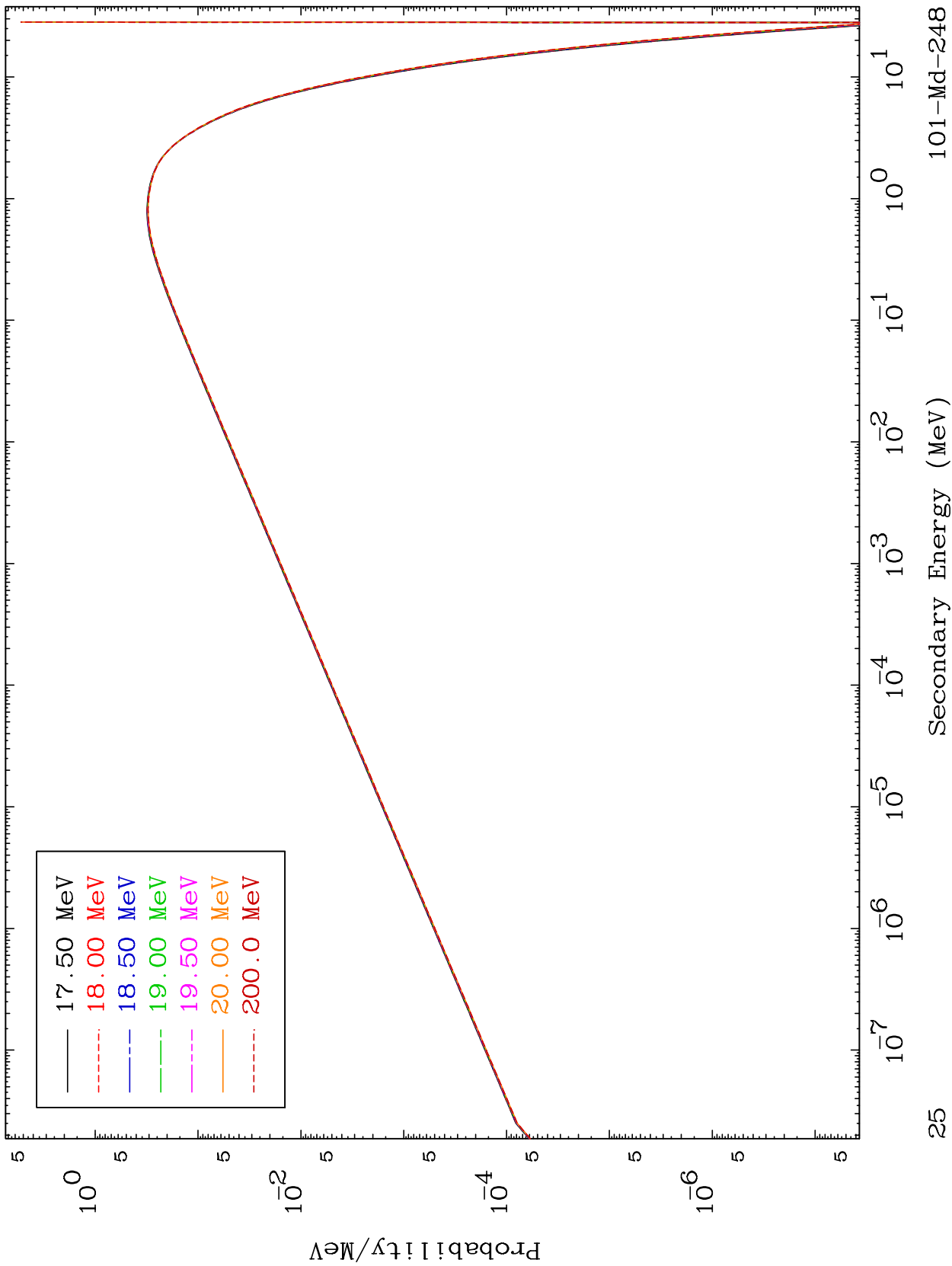
Fission
Energy Distribution







Fission
Energy Distribution

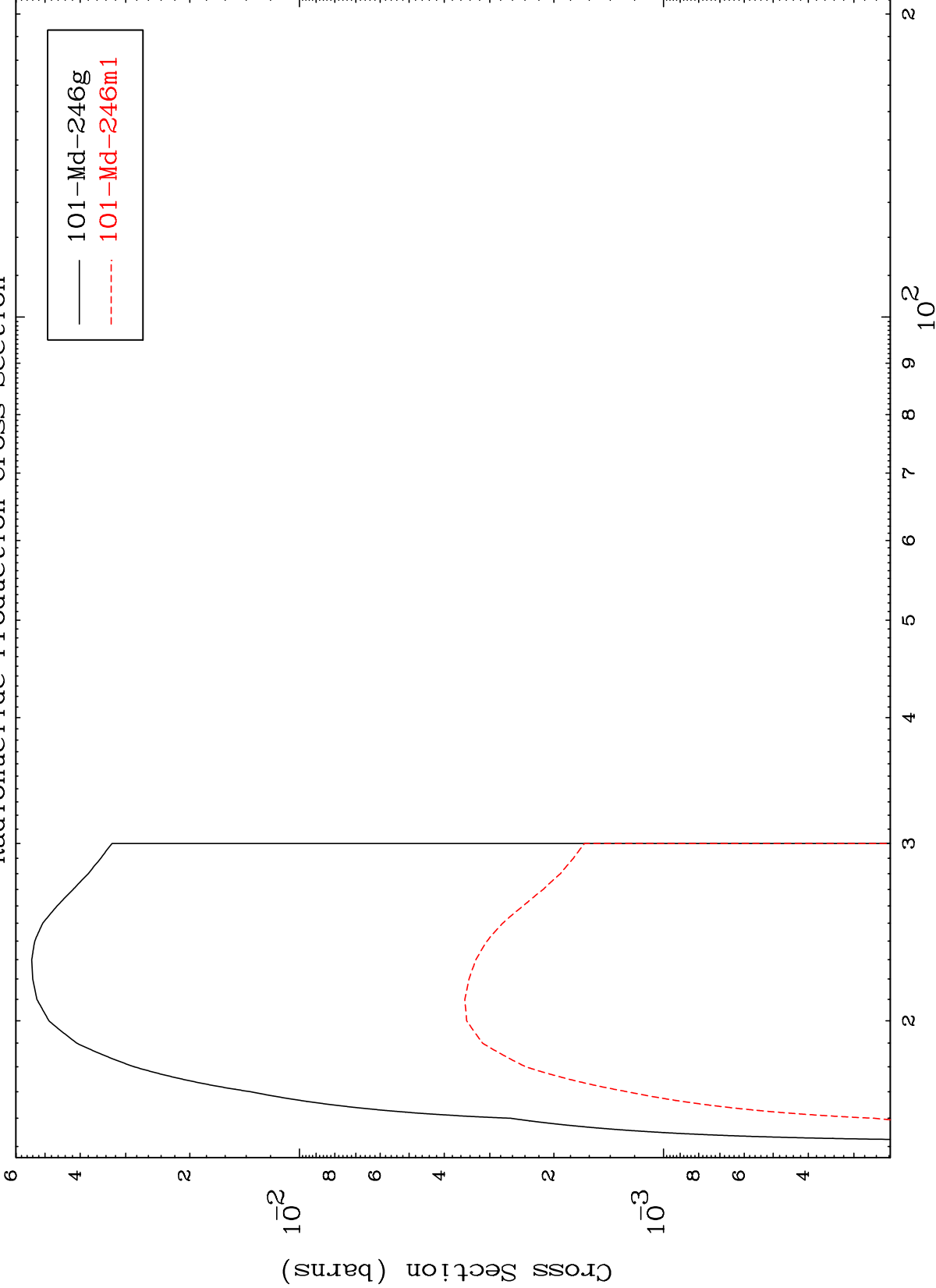


MAT 9949

(n,3n)

101-Md-248

Radionuclide Production Cross Section



26

Incident Energy (MeV)

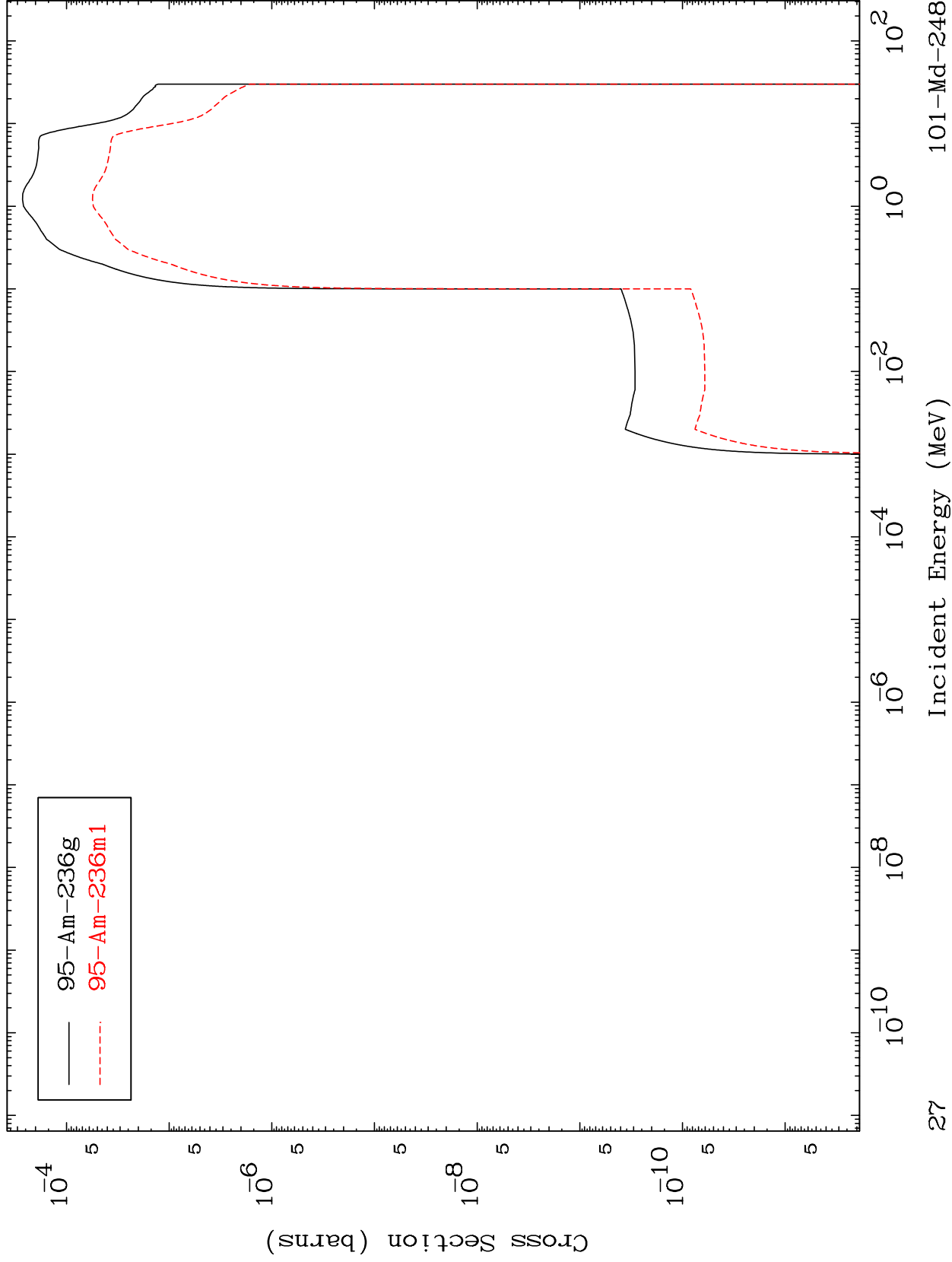
101-Md-248

MAT 9949

(n,n') 3 α

101-Md-248

Radionuclide Production Cross Section

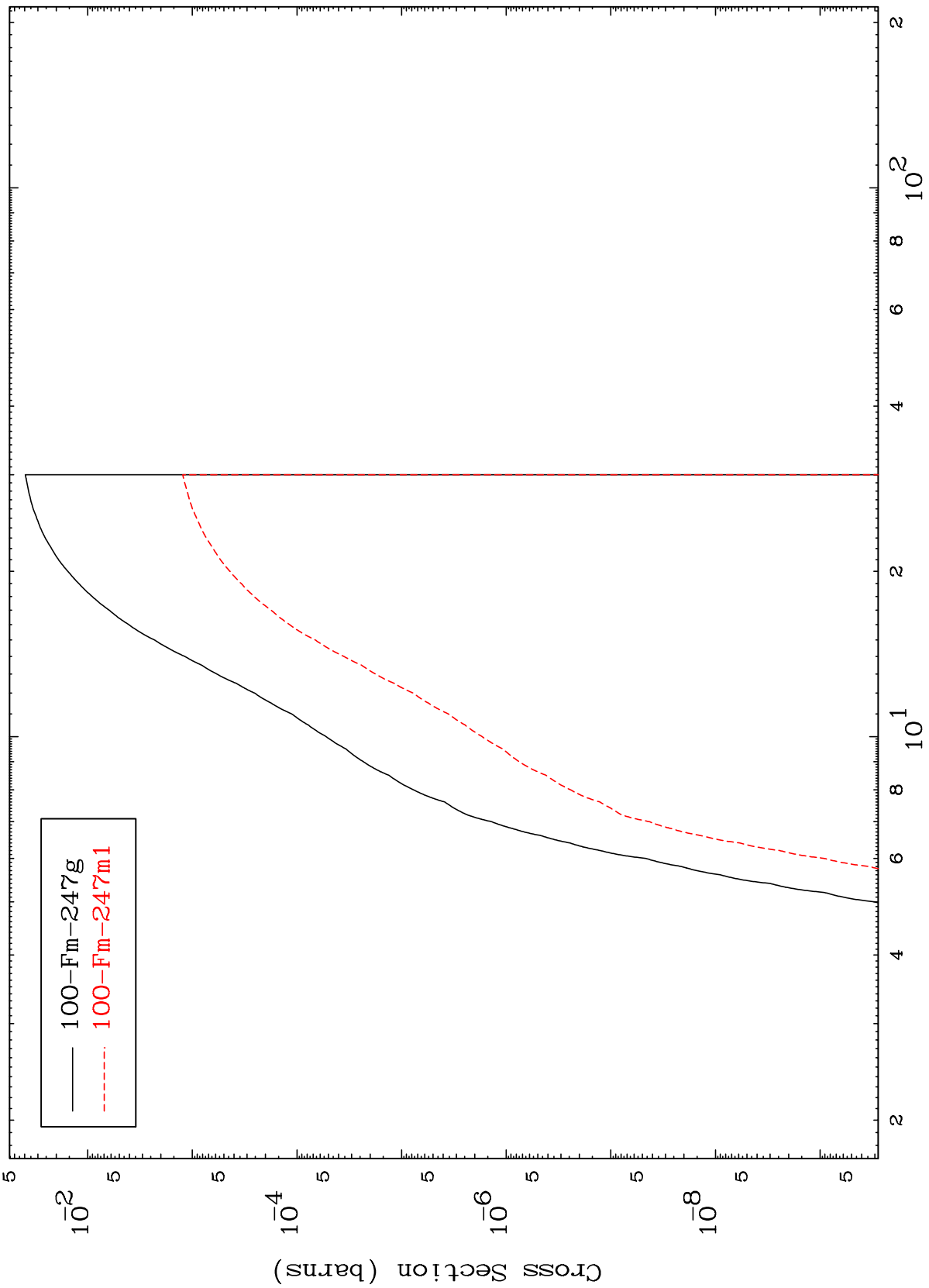


MAT 9949

(n,n') p

101-Md-248

Radionuclide Production Cross Section



28

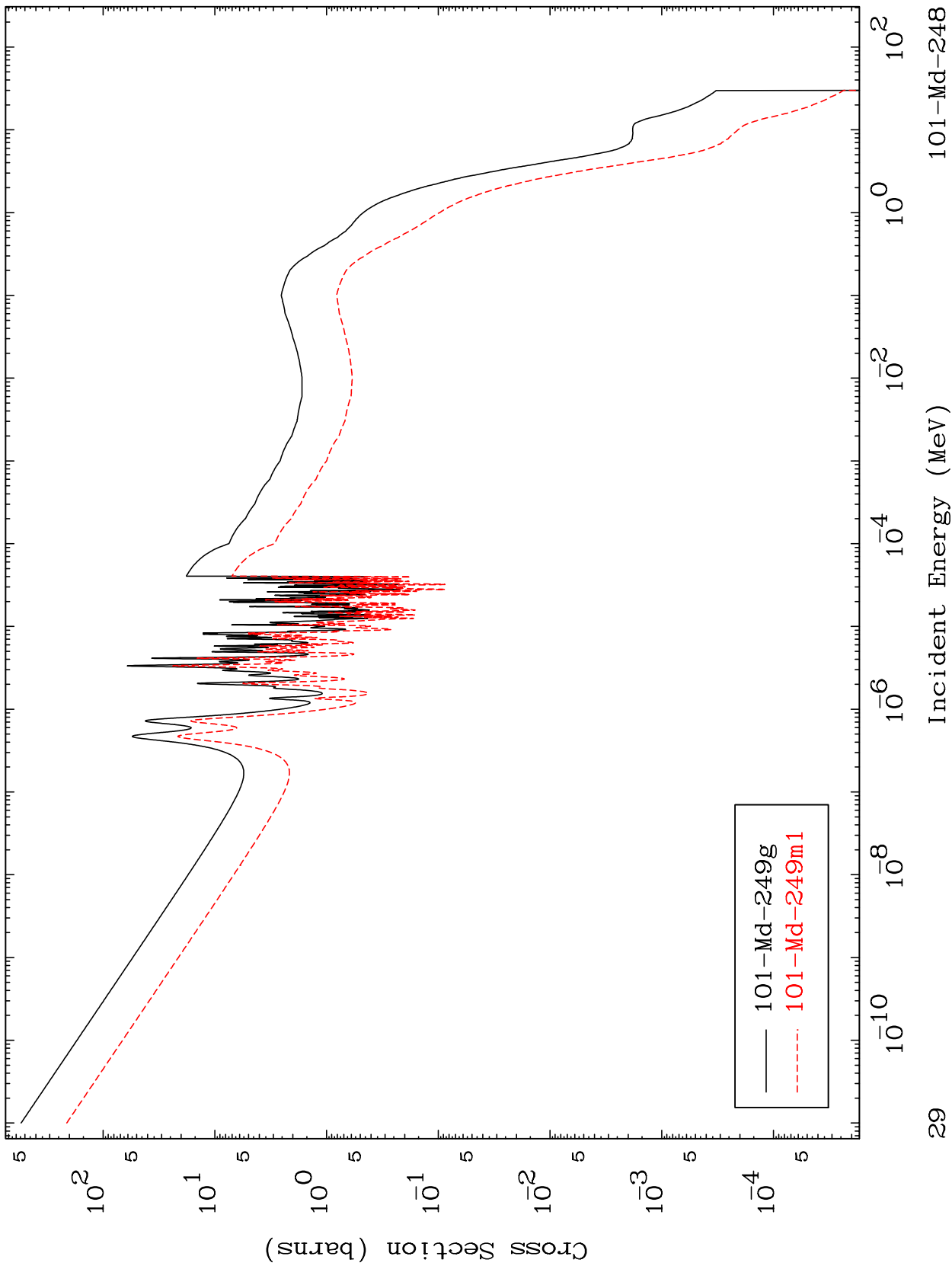
Incident Energy (MeV)

101-Md-248

MAT 9949

101-Md-248

(n, γ)
Radionuclide Production Cross Section



MAT 9949

(n,d)

101-Md-248

Radionuclide Production Cross Section

