

Program EVALPLOT
(Version 2015-2)

by

Dermott E. Cullen
(Present Contact Information)

Dermott E. Cullen
1466 Hudson Way
Livermore, CA 94550
U.S.A.

Tele: 925-443-1911

E.Mail:redcullen1@comcast.net
Web:home.comcast.net/~redcullen1

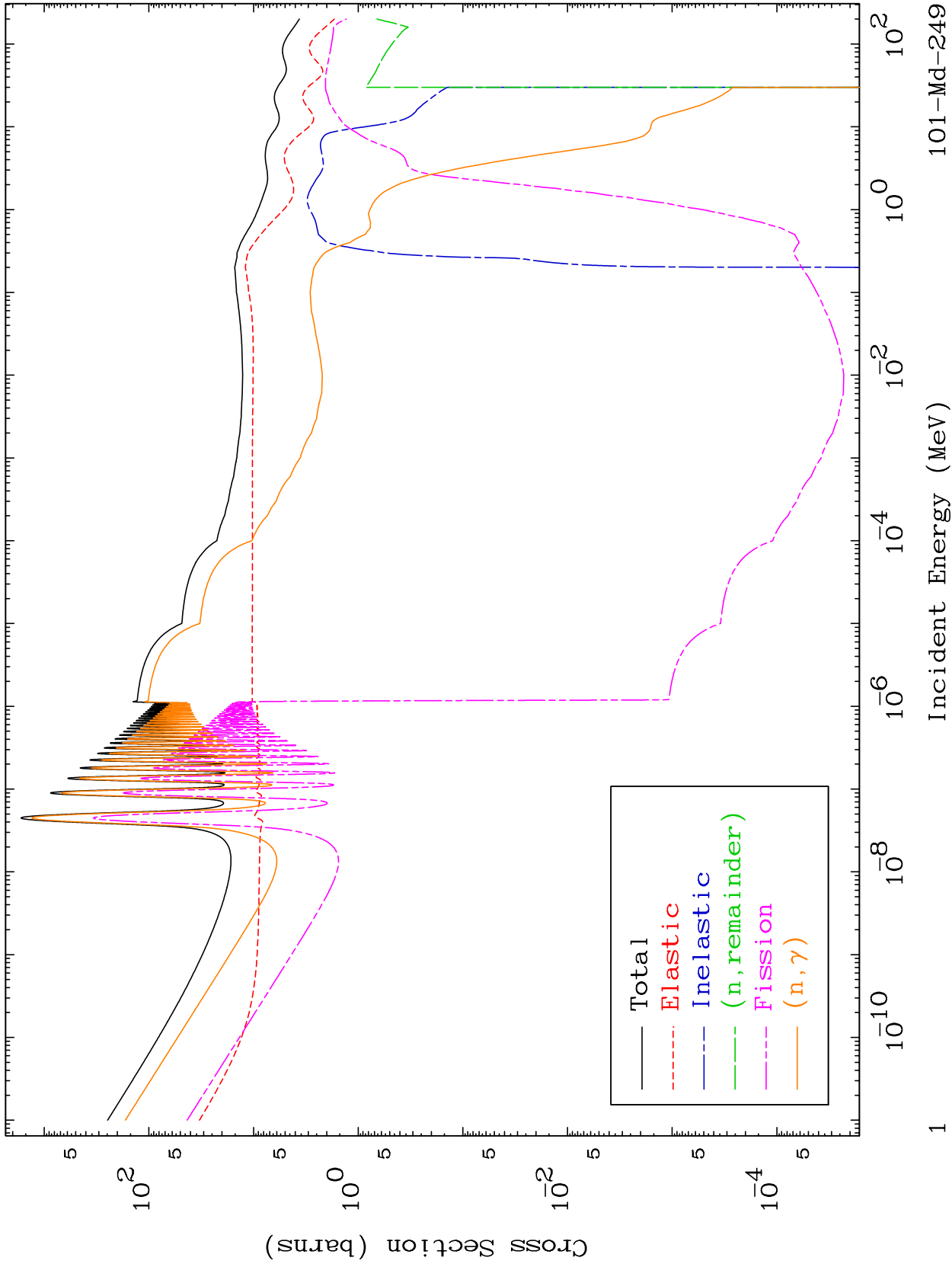
Press Mouse Button to Start

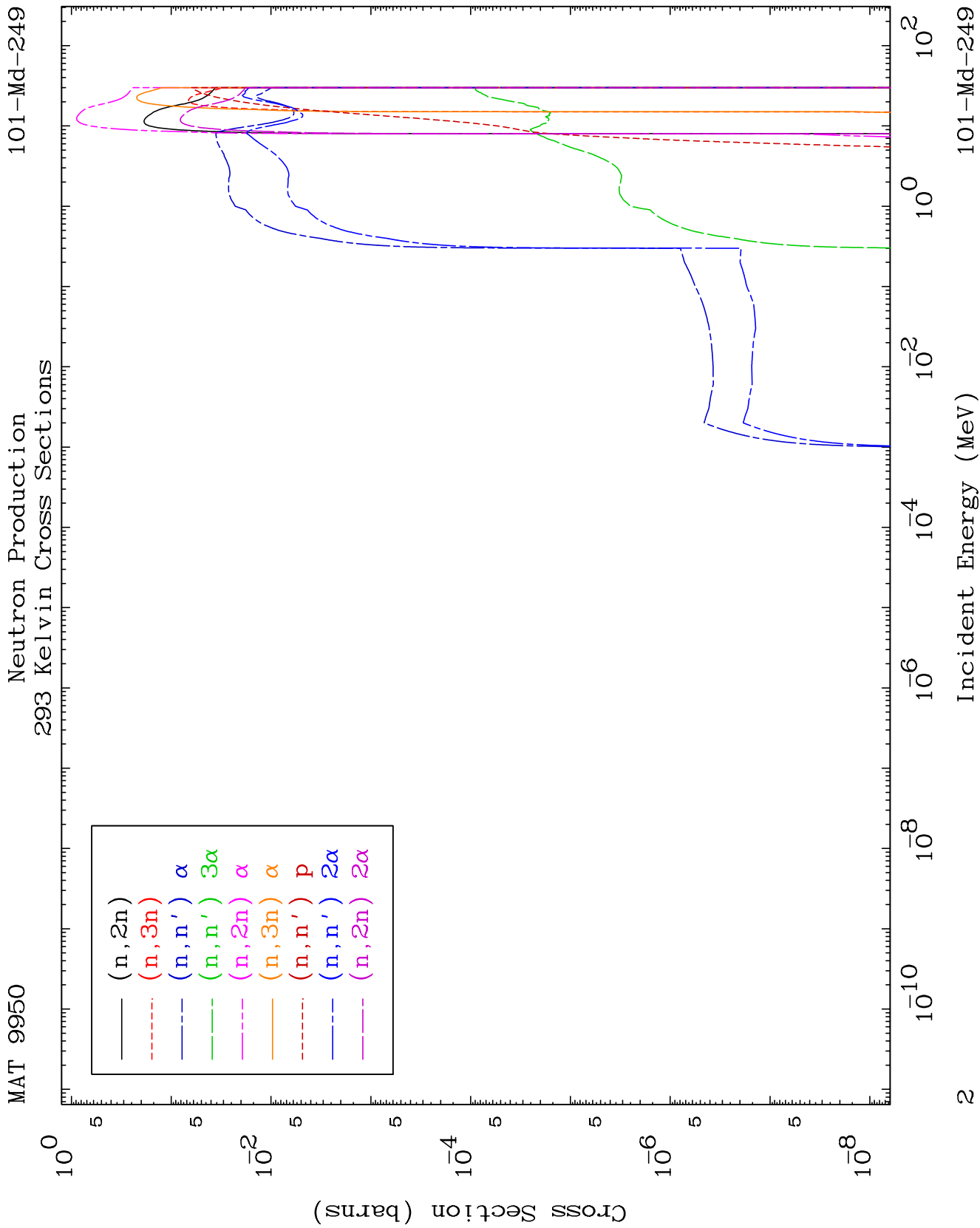
MAT 9950

Major

293 Kelvin Cross Sections

101-Md-249

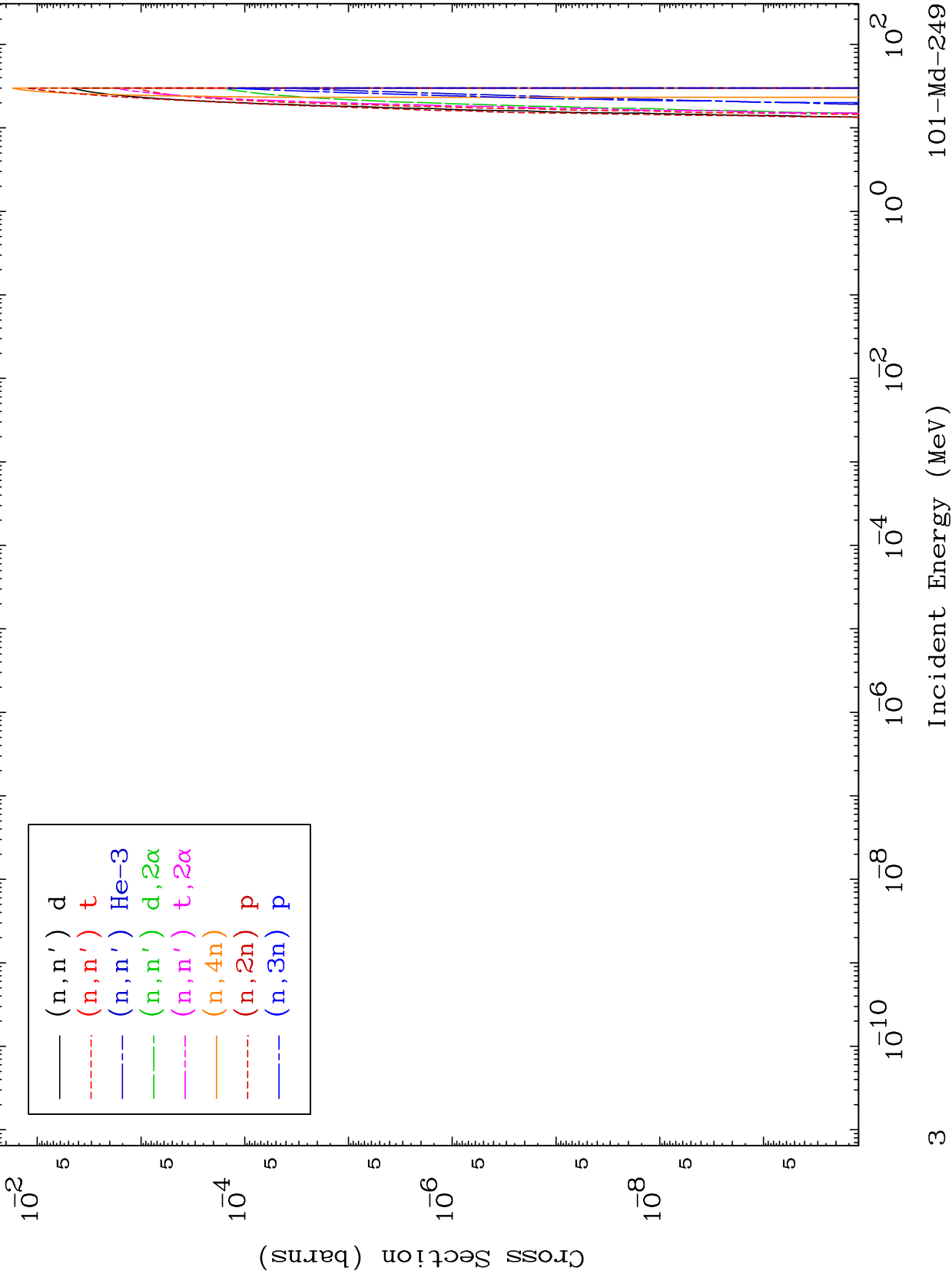


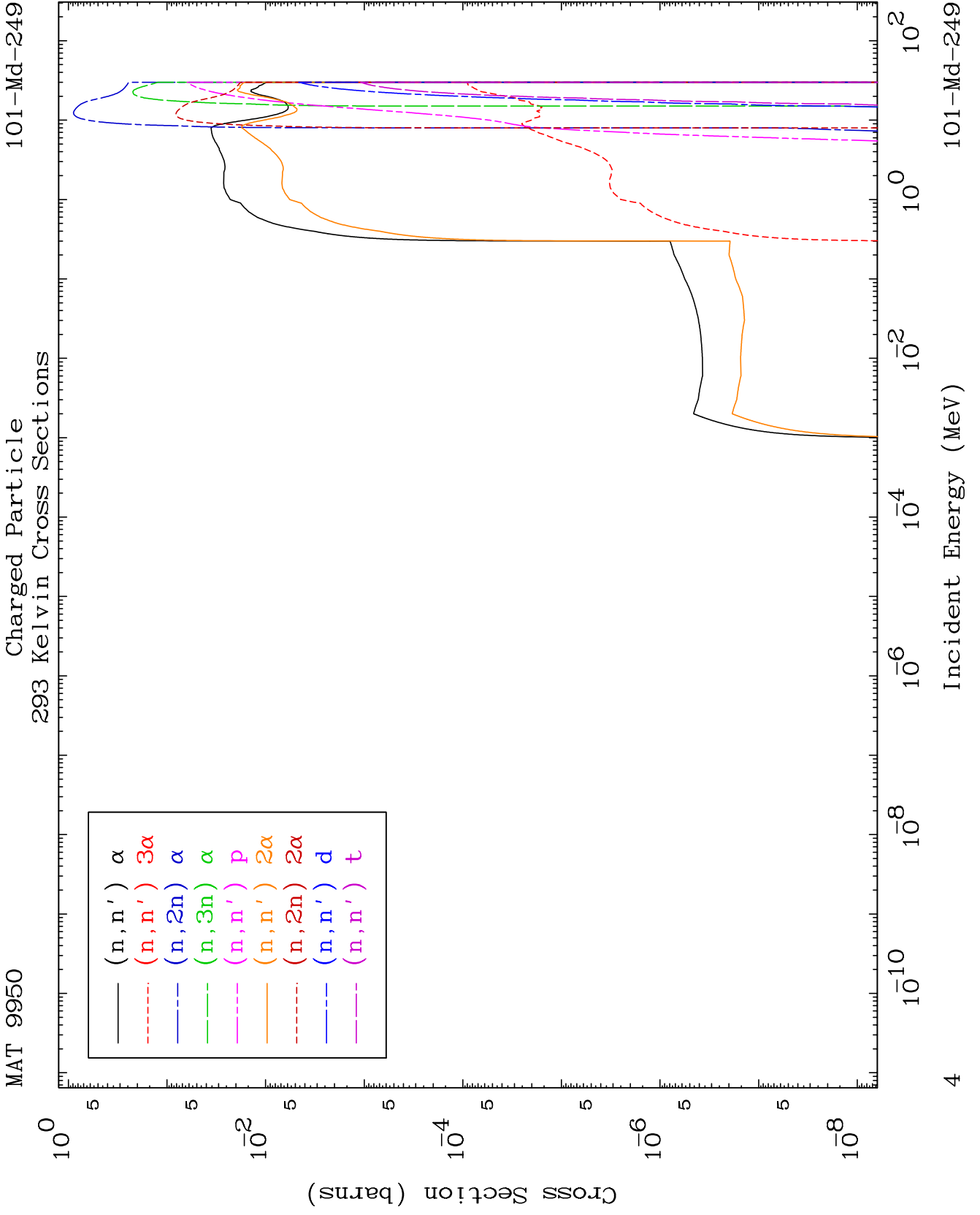


MAT 9950

Neutron Production
293 Kelvin Cross Sections

101-Md-249

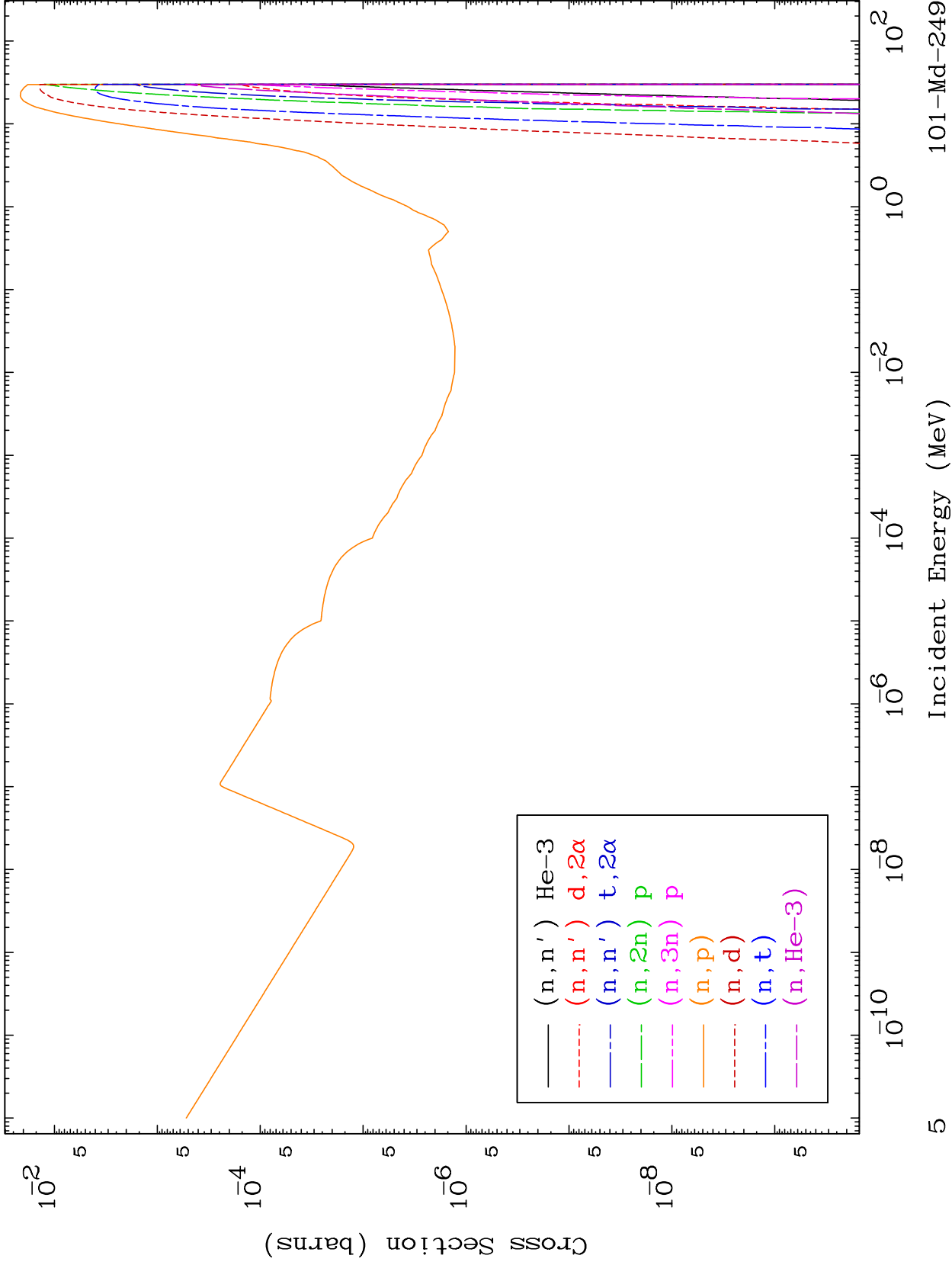




MAT 9950

Charged Particle
293 Kelvin Cross Sections

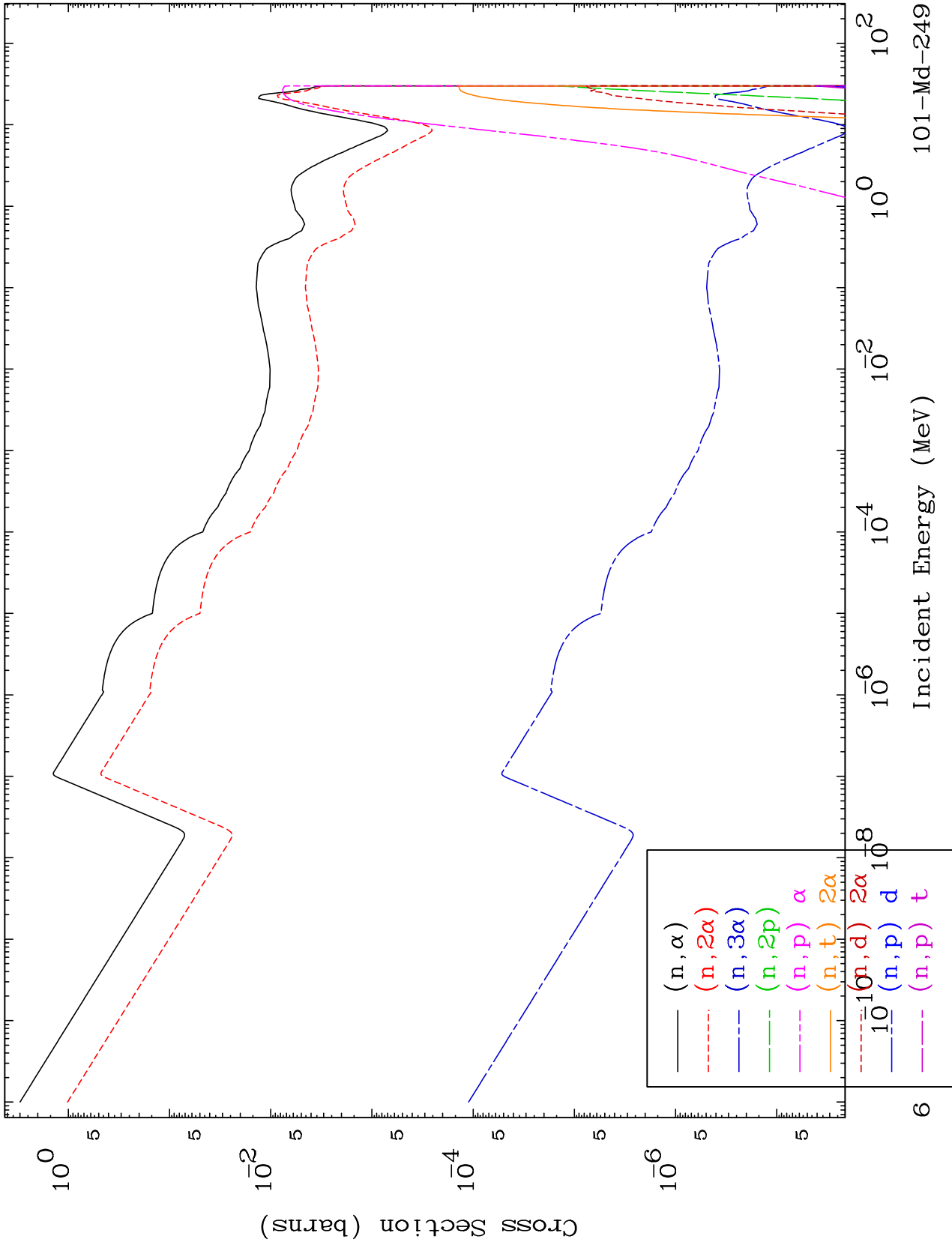
101-Md-249



MAT 9950

Charged Particle
293 Kelvin Cross Sections

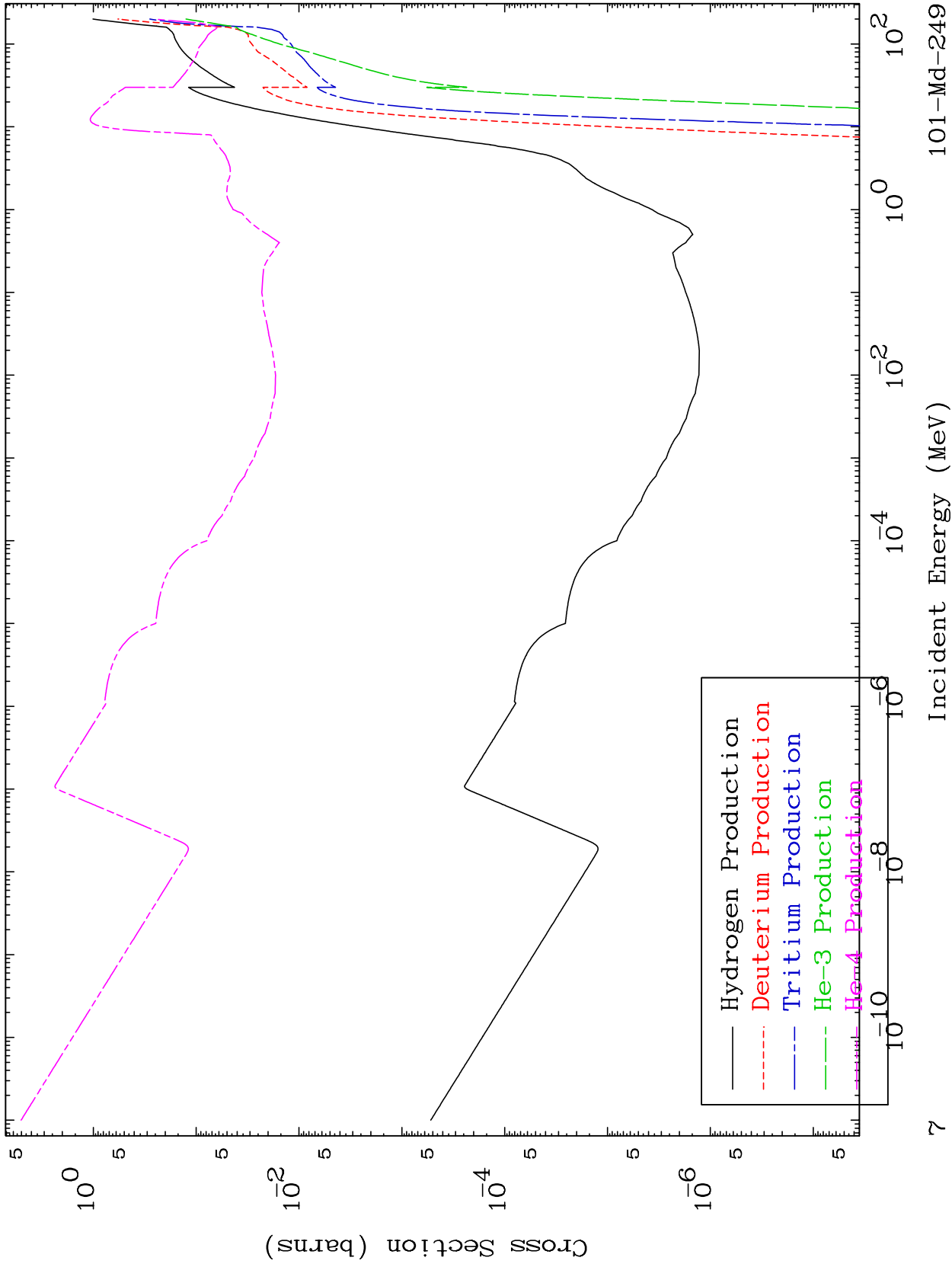
101-Md-249



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Particle Production
293 Kelvin Cross Sections

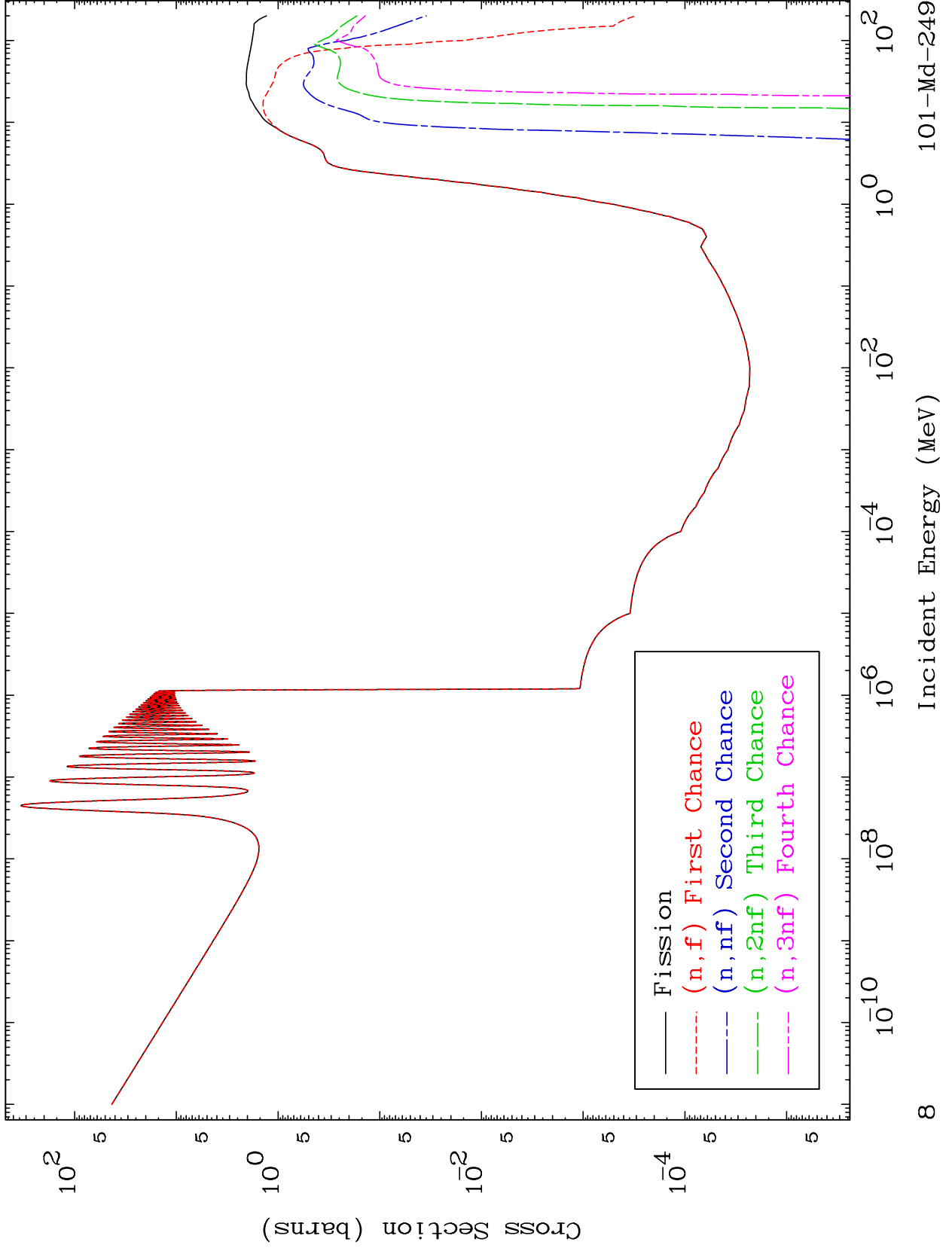
101-Md-249



MAT 9950

Fission
293 Kelvin Cross Sections

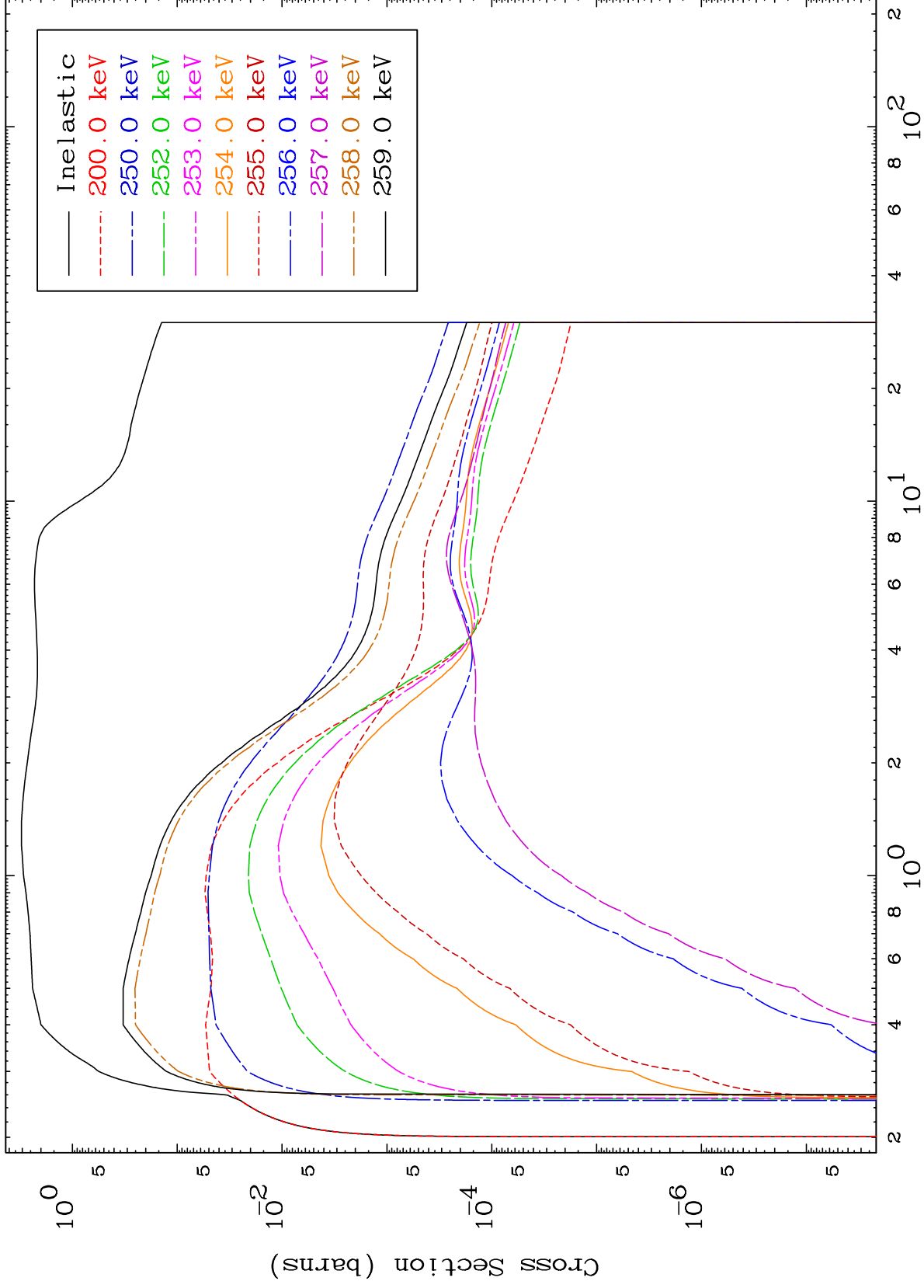
101-Md-249



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(n,n') Level
293 Kelvin Cross Sections

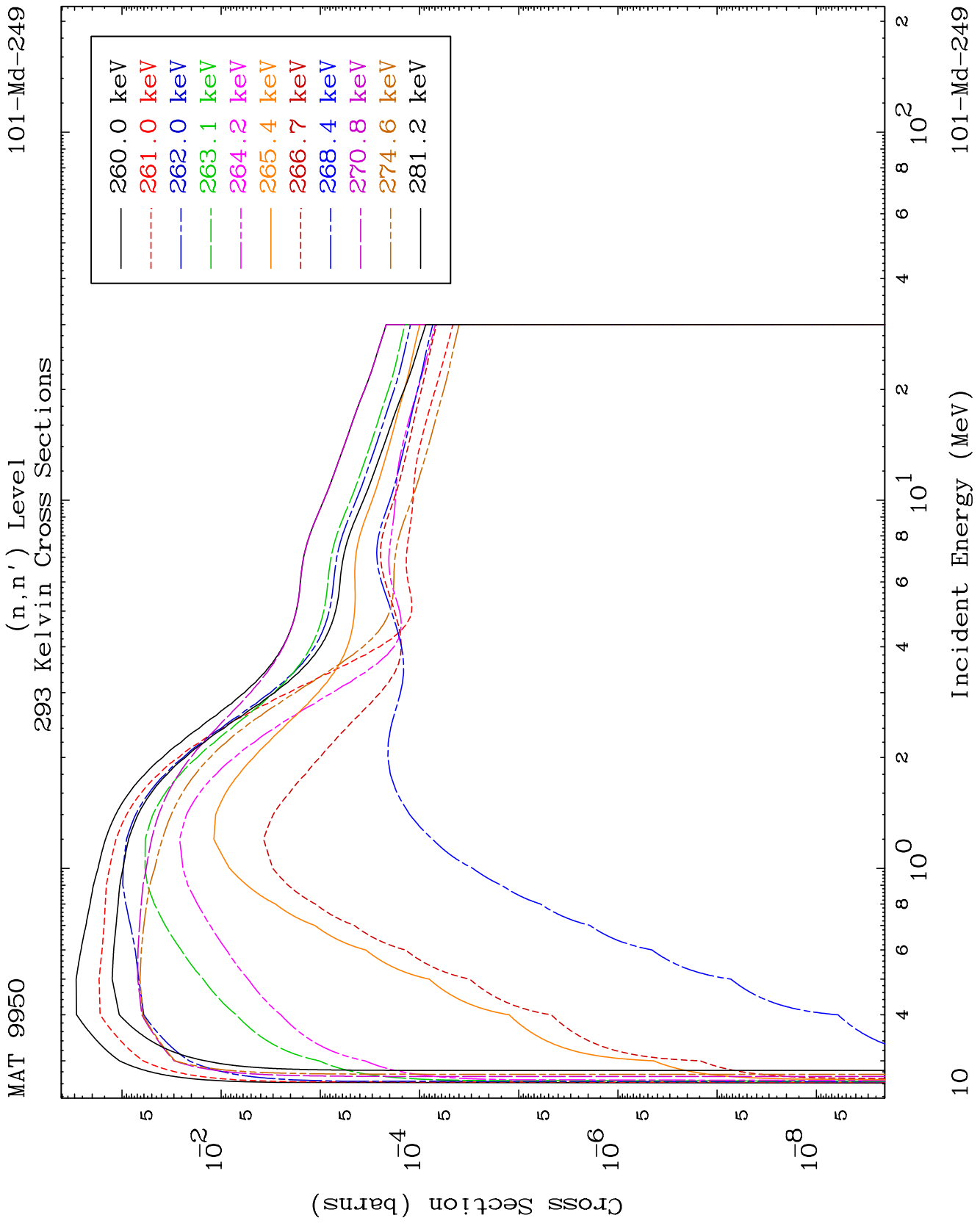
101-Md-249

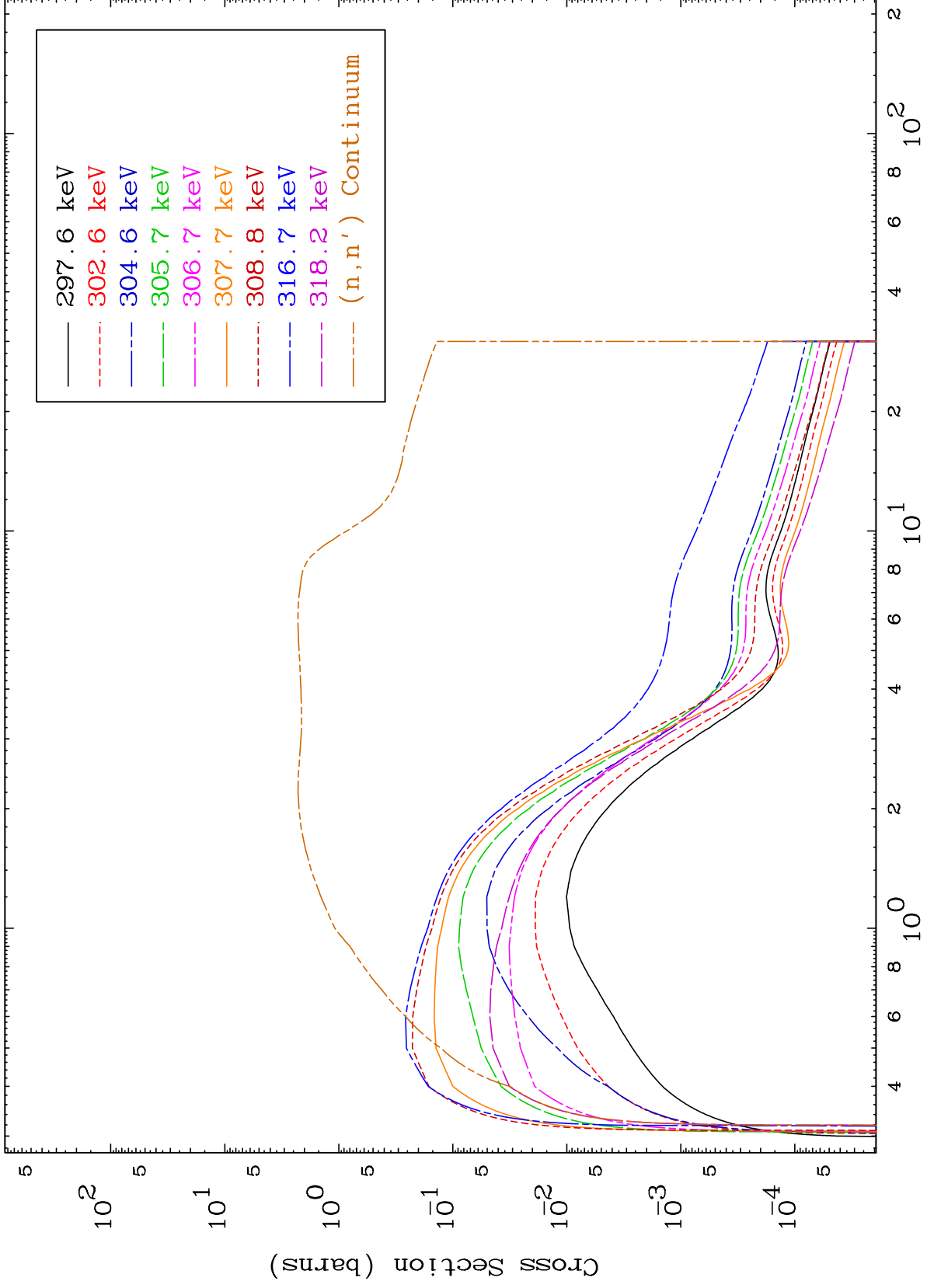


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Incident Energy (MeV)

101-Md-249

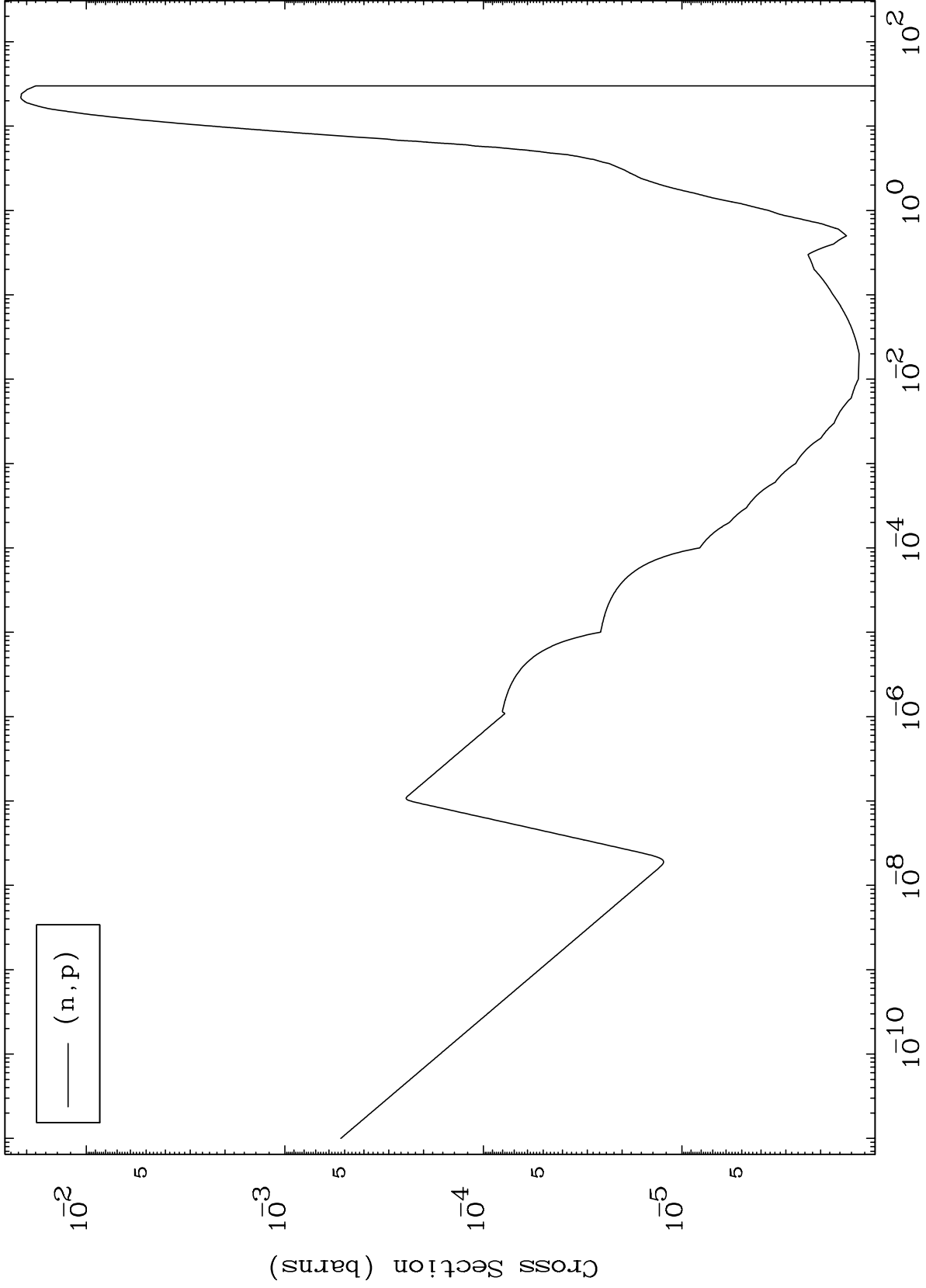




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(n,p) Levels
293 Kelvin Cross Sections

101-Md-249



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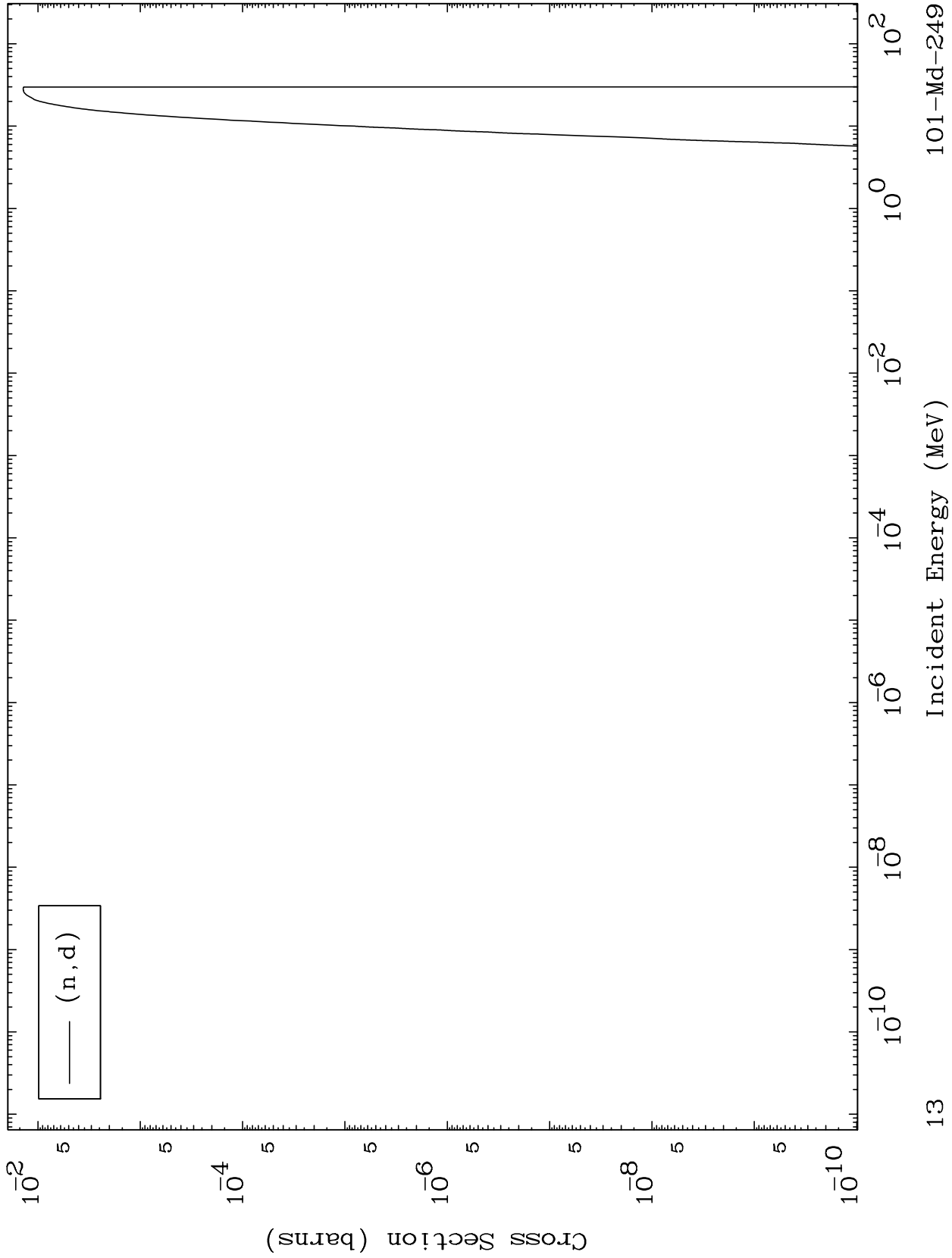
Incident Energy (MeV)

101-Md-249

MAT 9950

(n,d) Levels
293 Kelvin Cross Sections

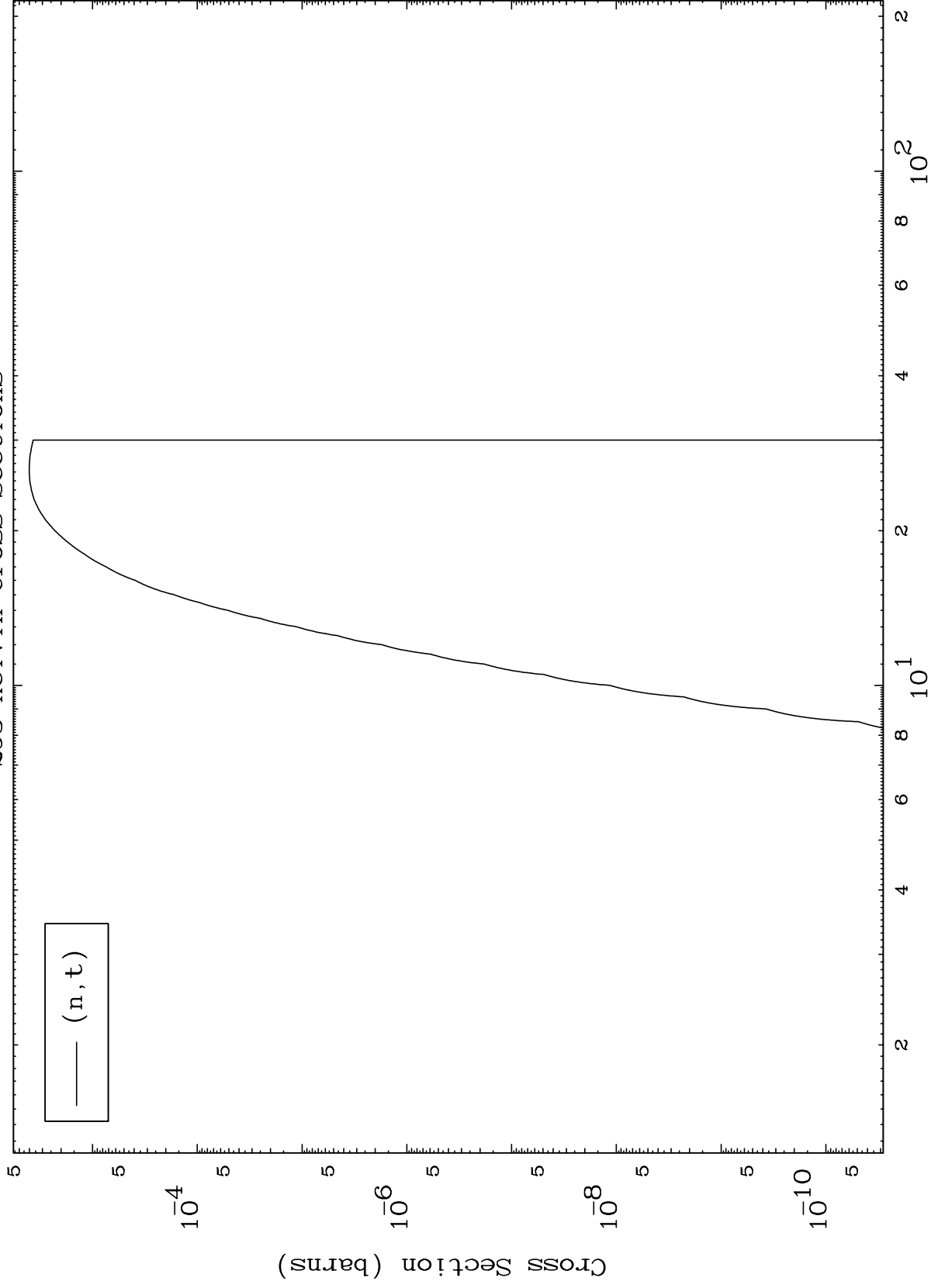
101-Md-249



MAT 9950

(n,t) Levels
293 Kelvin Cross Sections

101-Md-249



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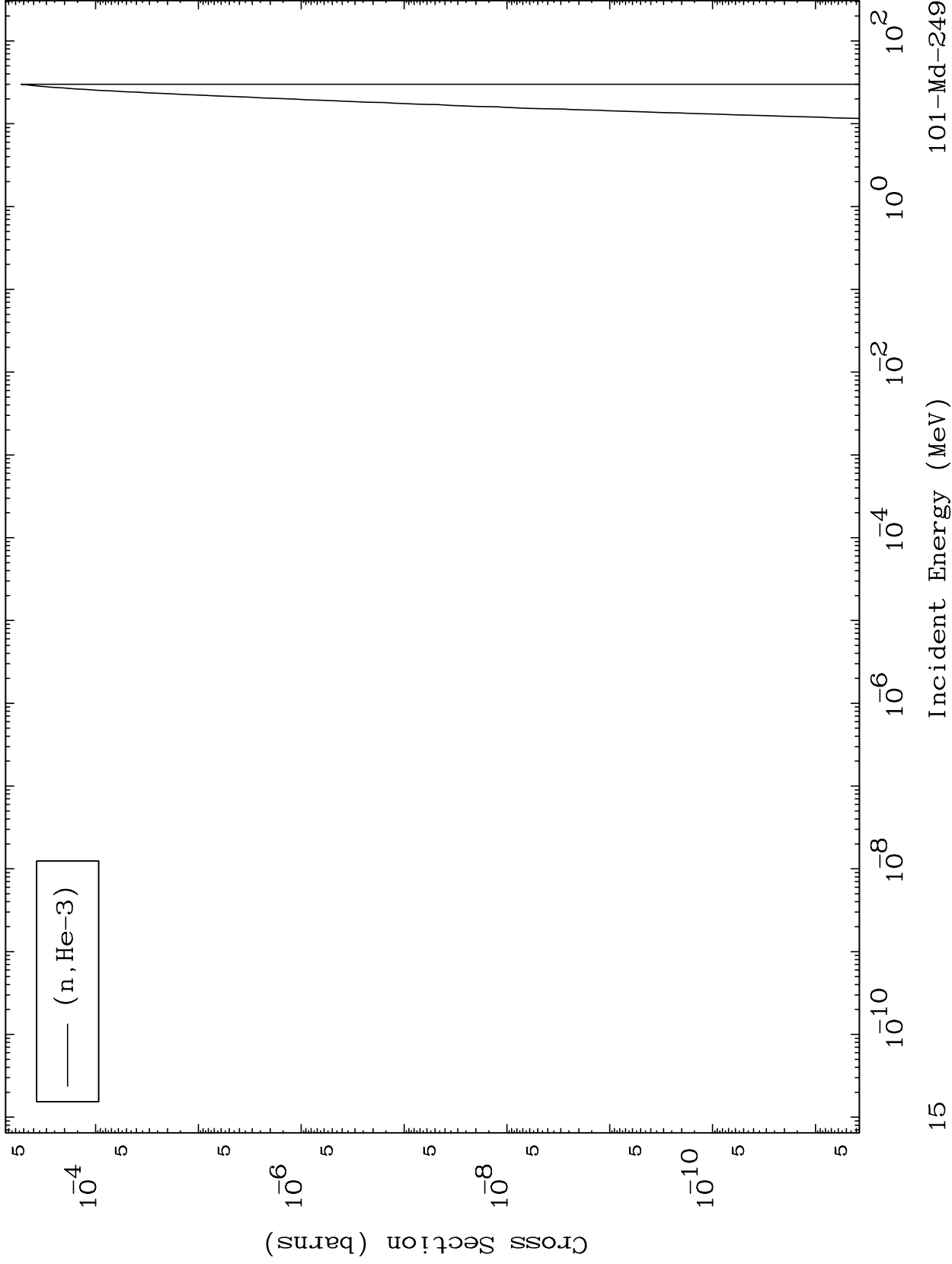
Incident Energy (MeV)

101-Md-249

MAT 9950

(n,He3) Levels
293 Kelvin Cross Sections

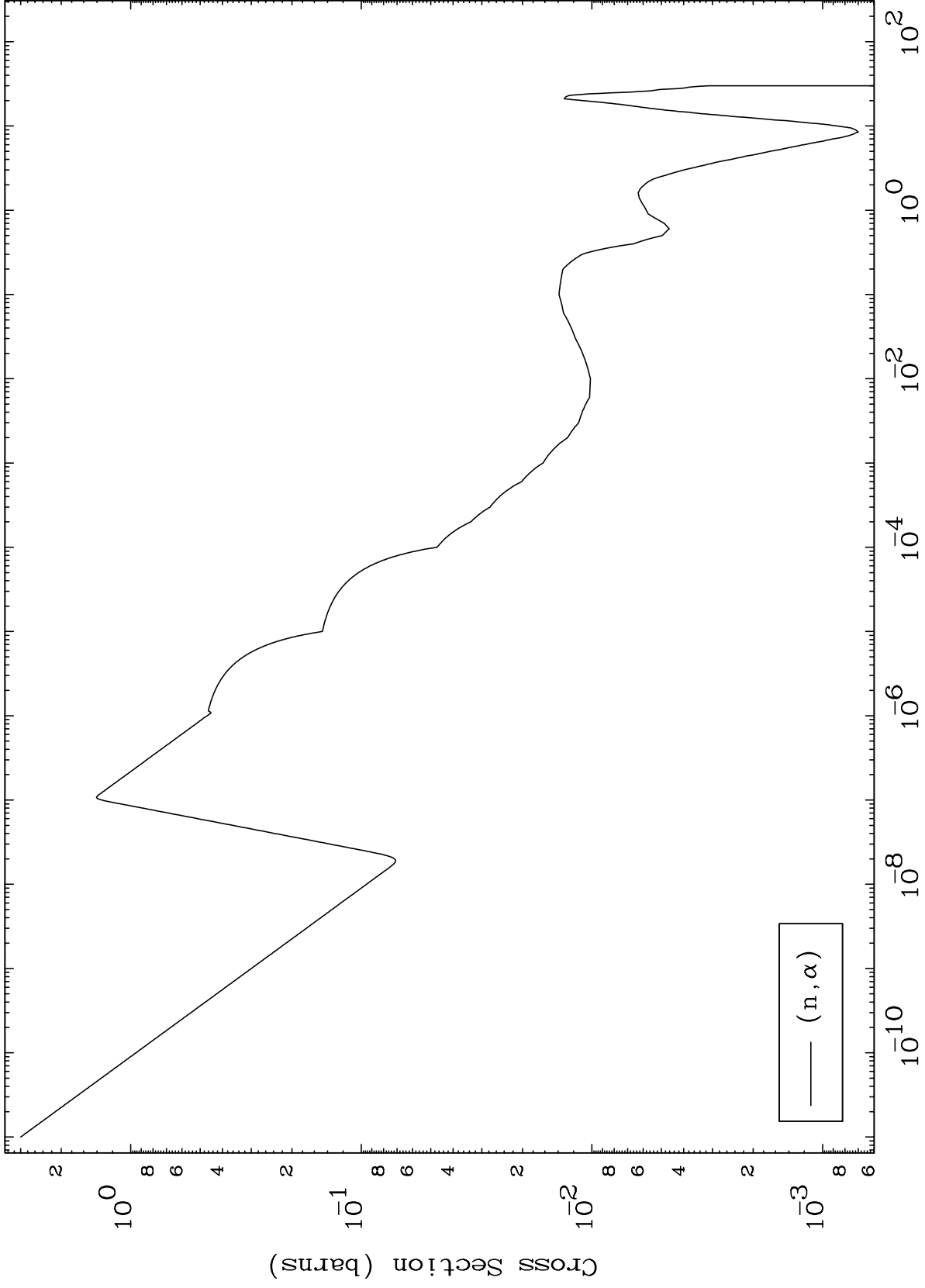
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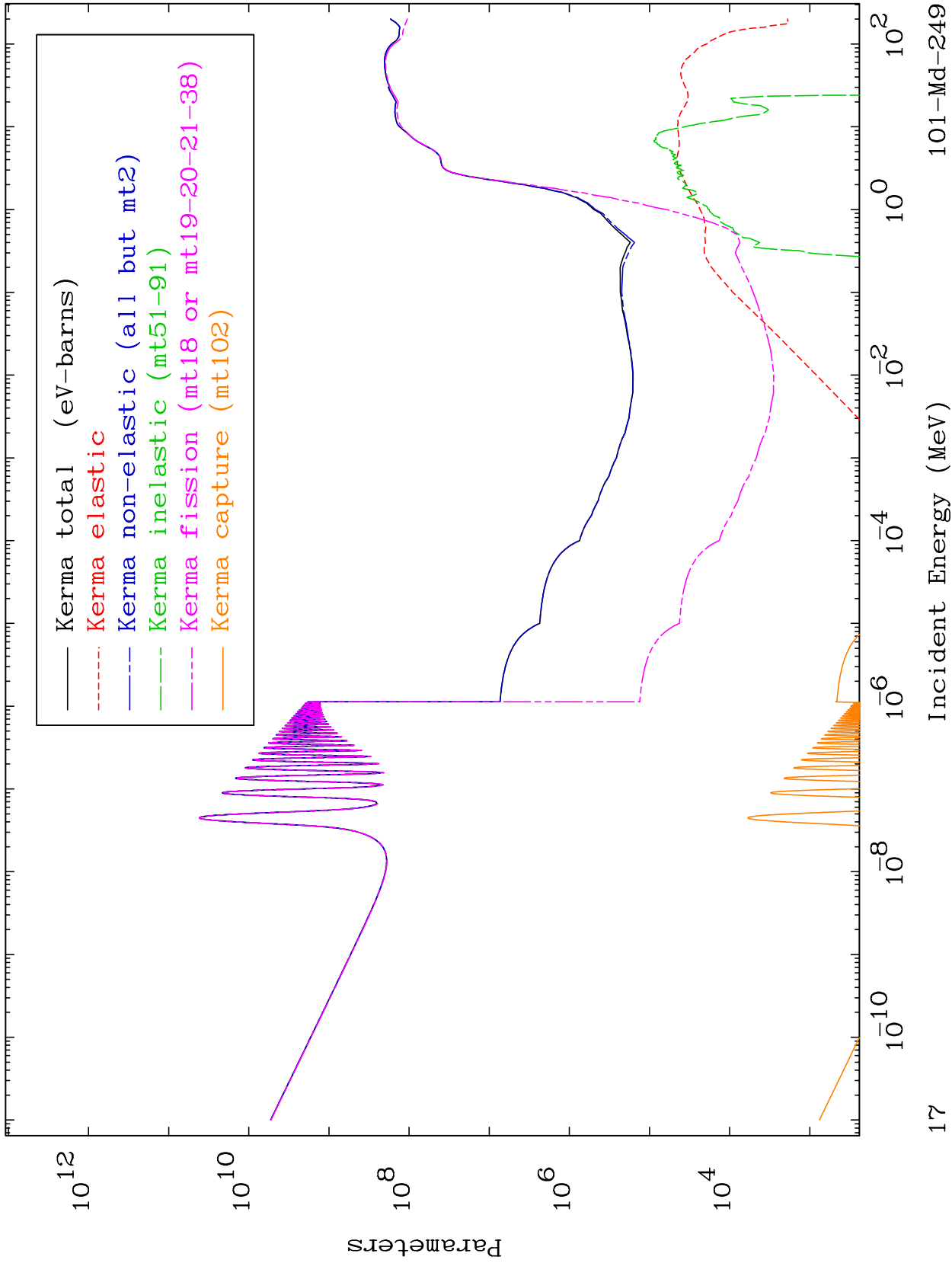


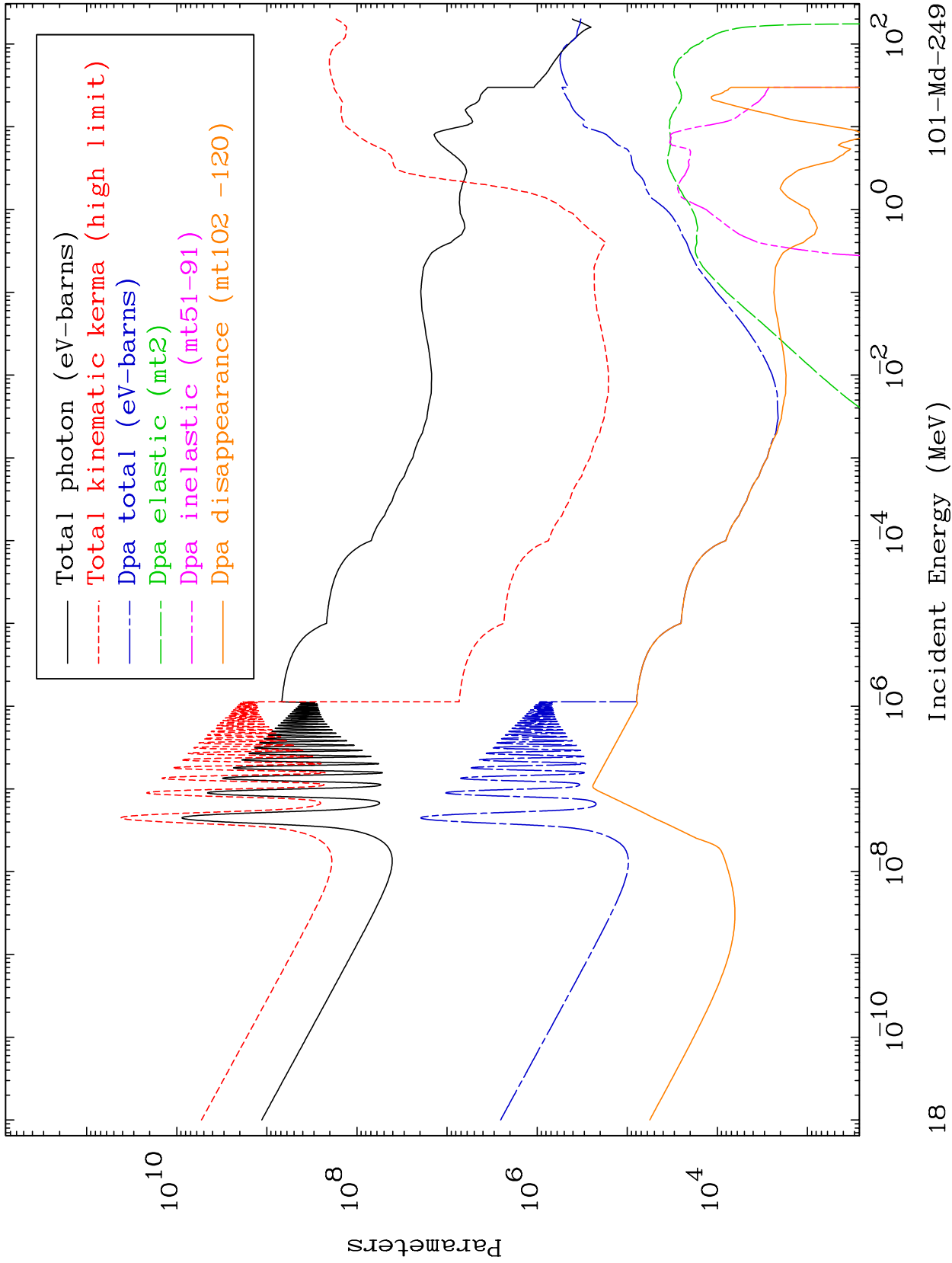
MAT 9950

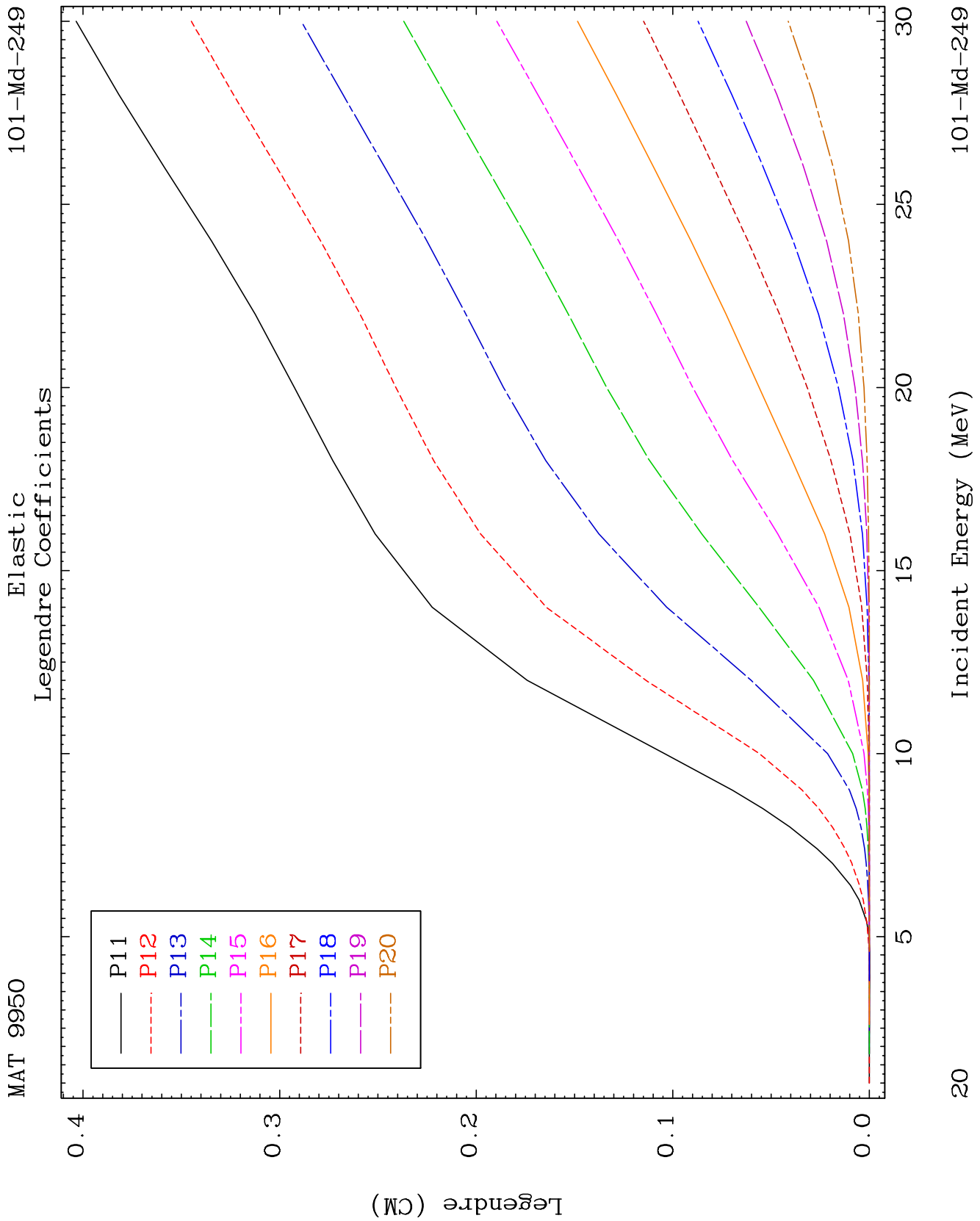
(n,α) Levels
293 Kelvin Cross Sections

101-Md-249





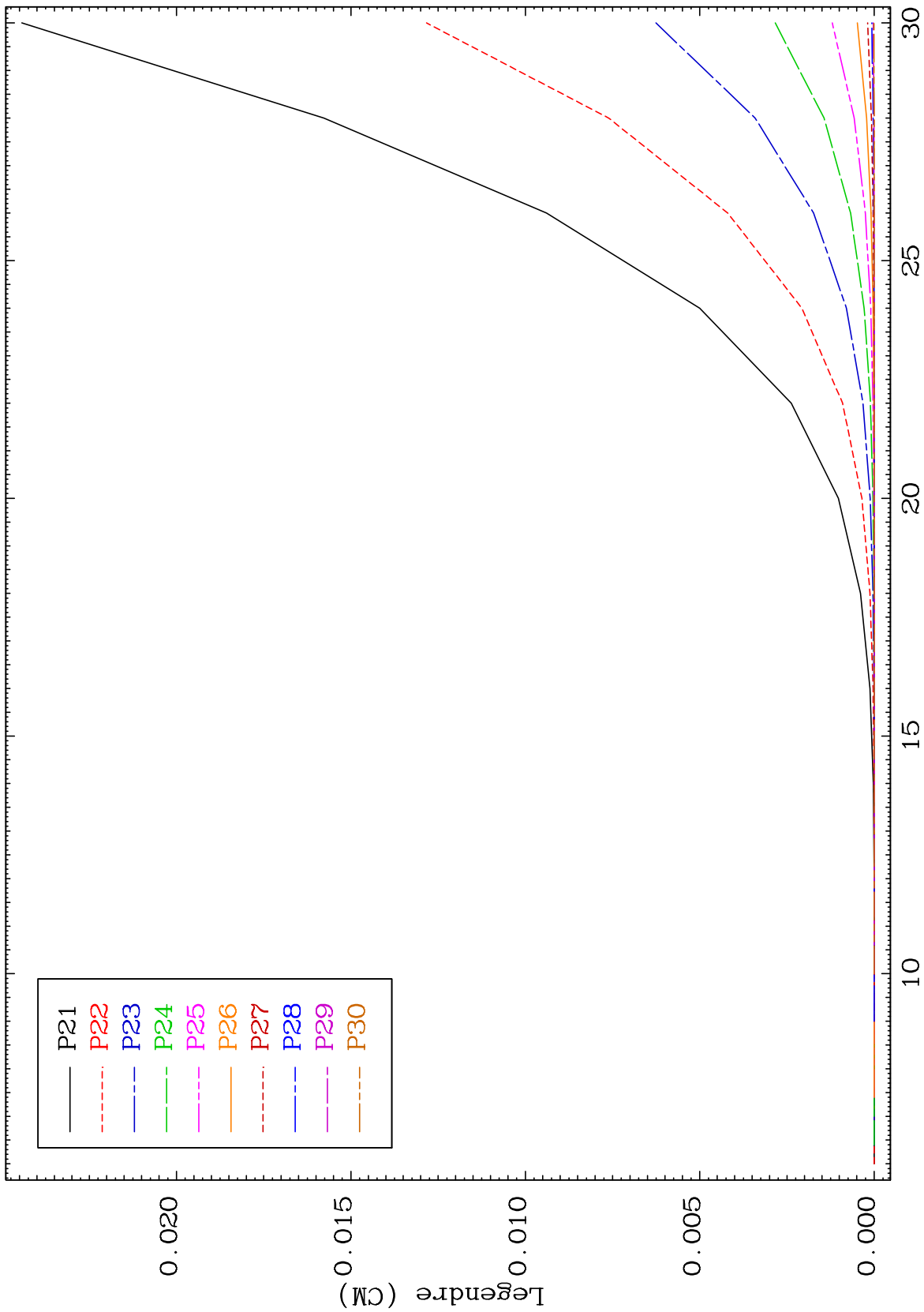




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Elastic Legendre Coefficients

101-Md-249



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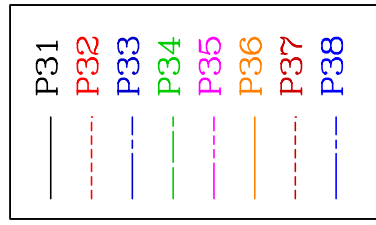
Incident Energy (MeV)

101-Md-249

MAT 9950

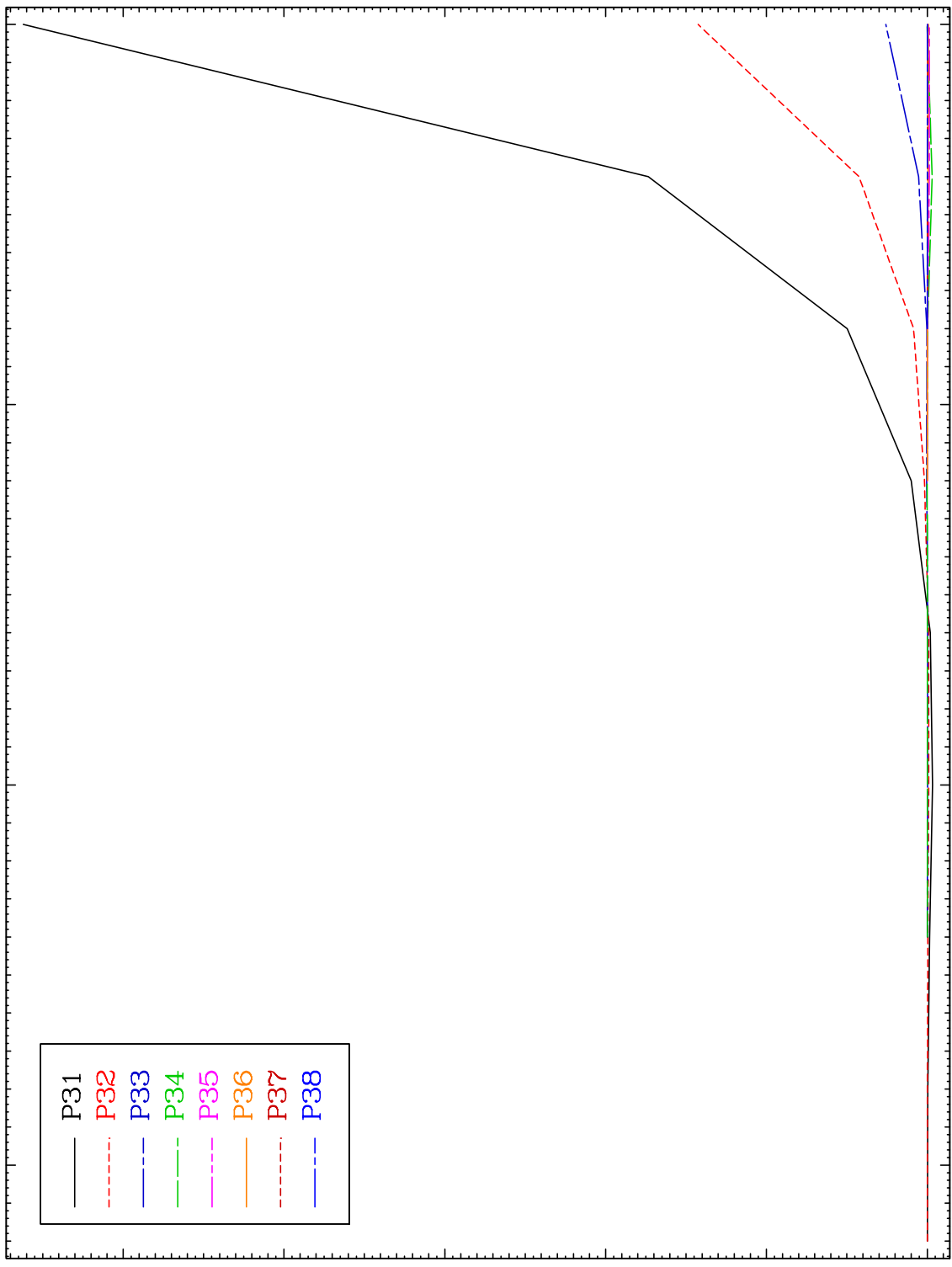
Elastic Legendre Coefficients

101-Md-249



$\times 10^{-6}$

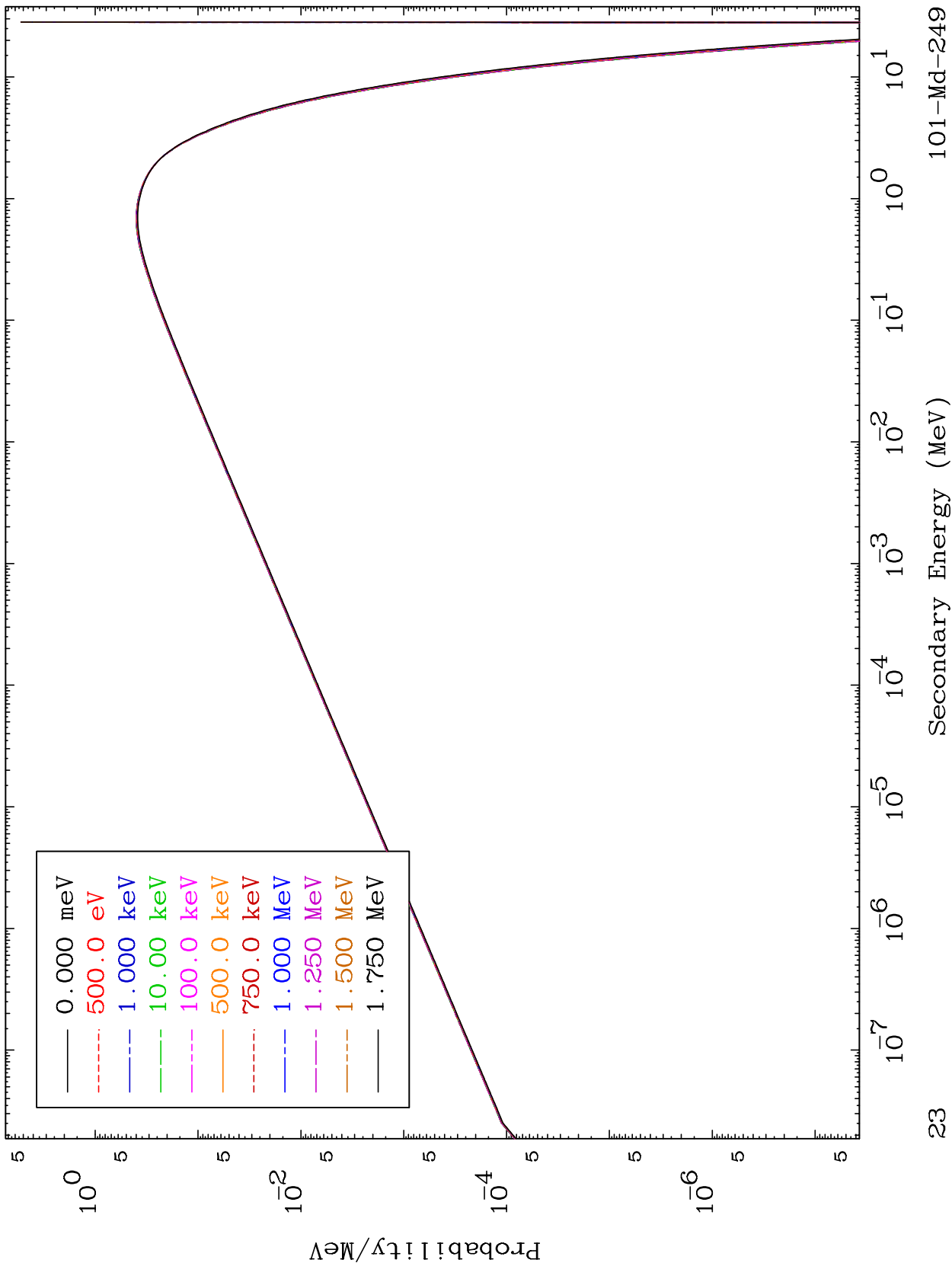
Legendre (CM)



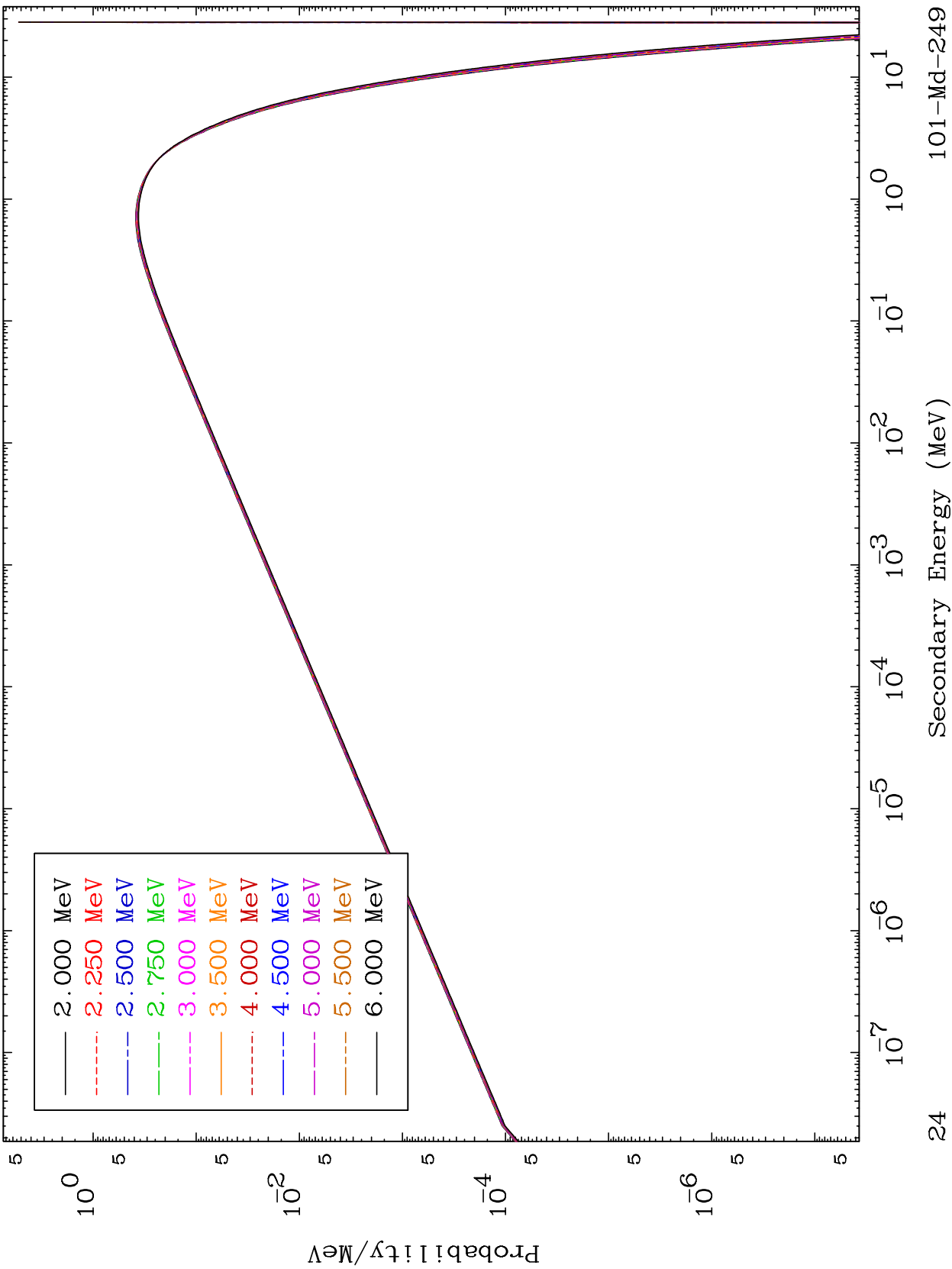
22

Incident Energy (MeV)

101-Md-249



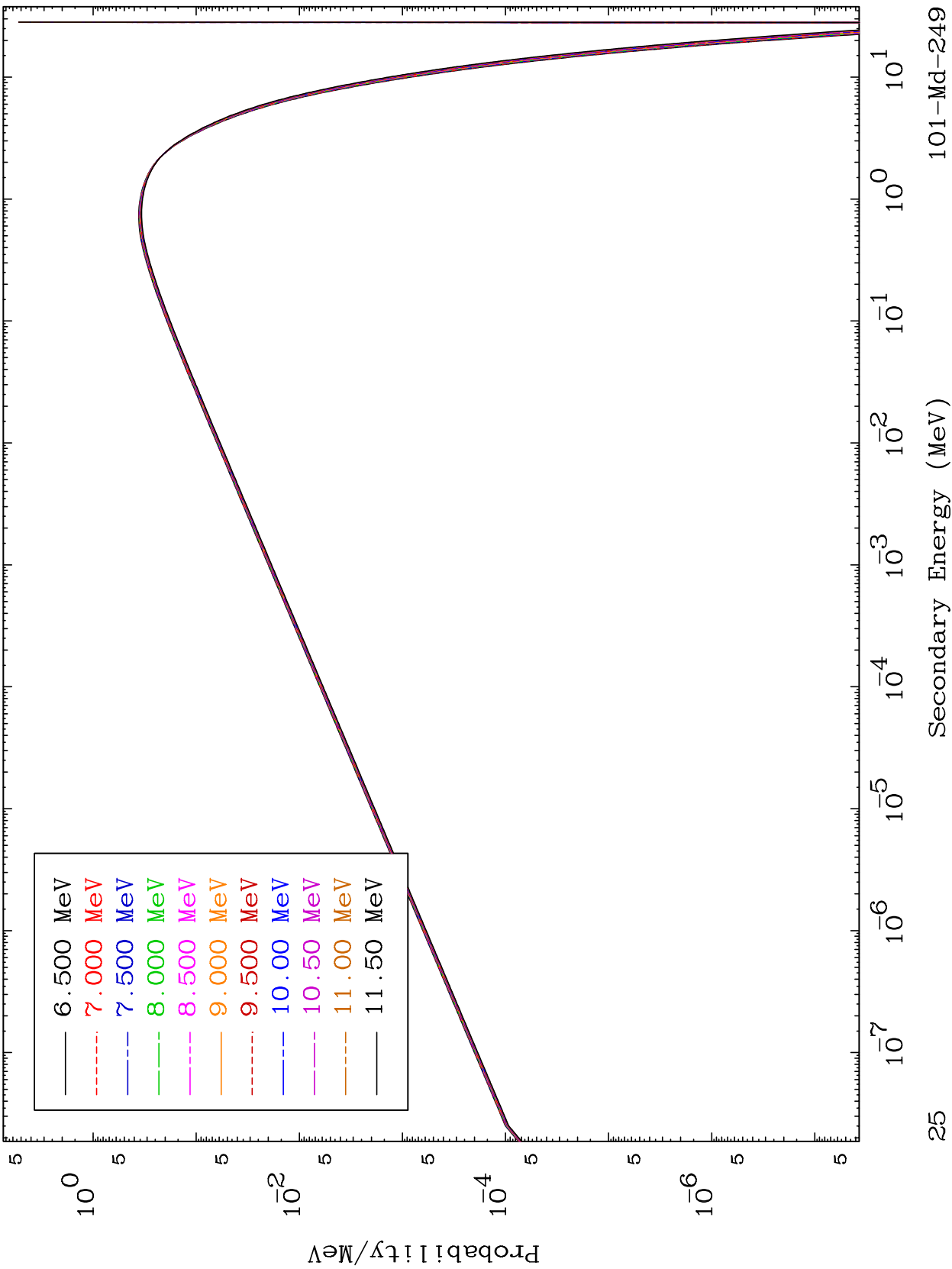
Fission
Energy Distribution



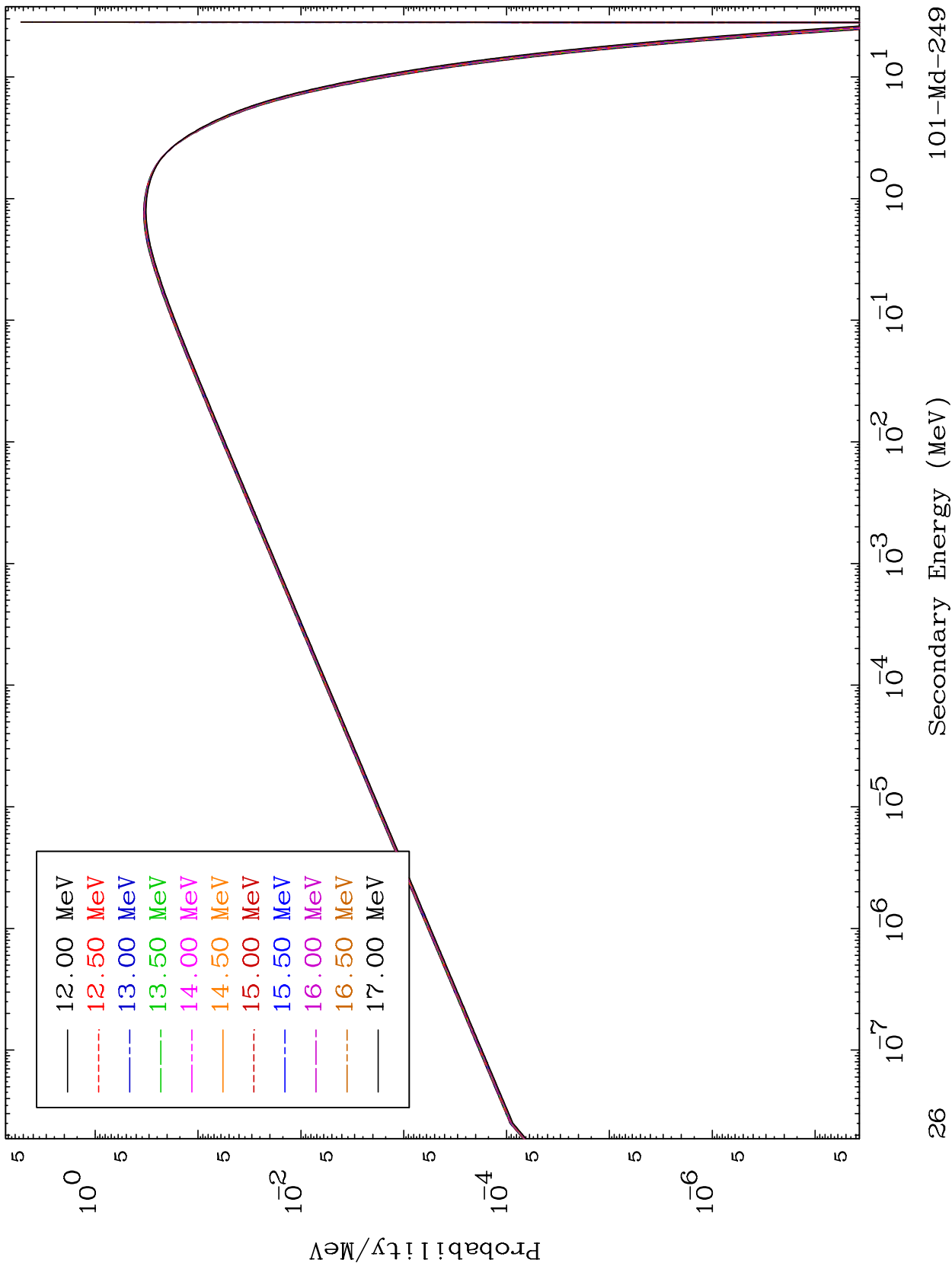
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Fission Energy Distribution

101-Md-249



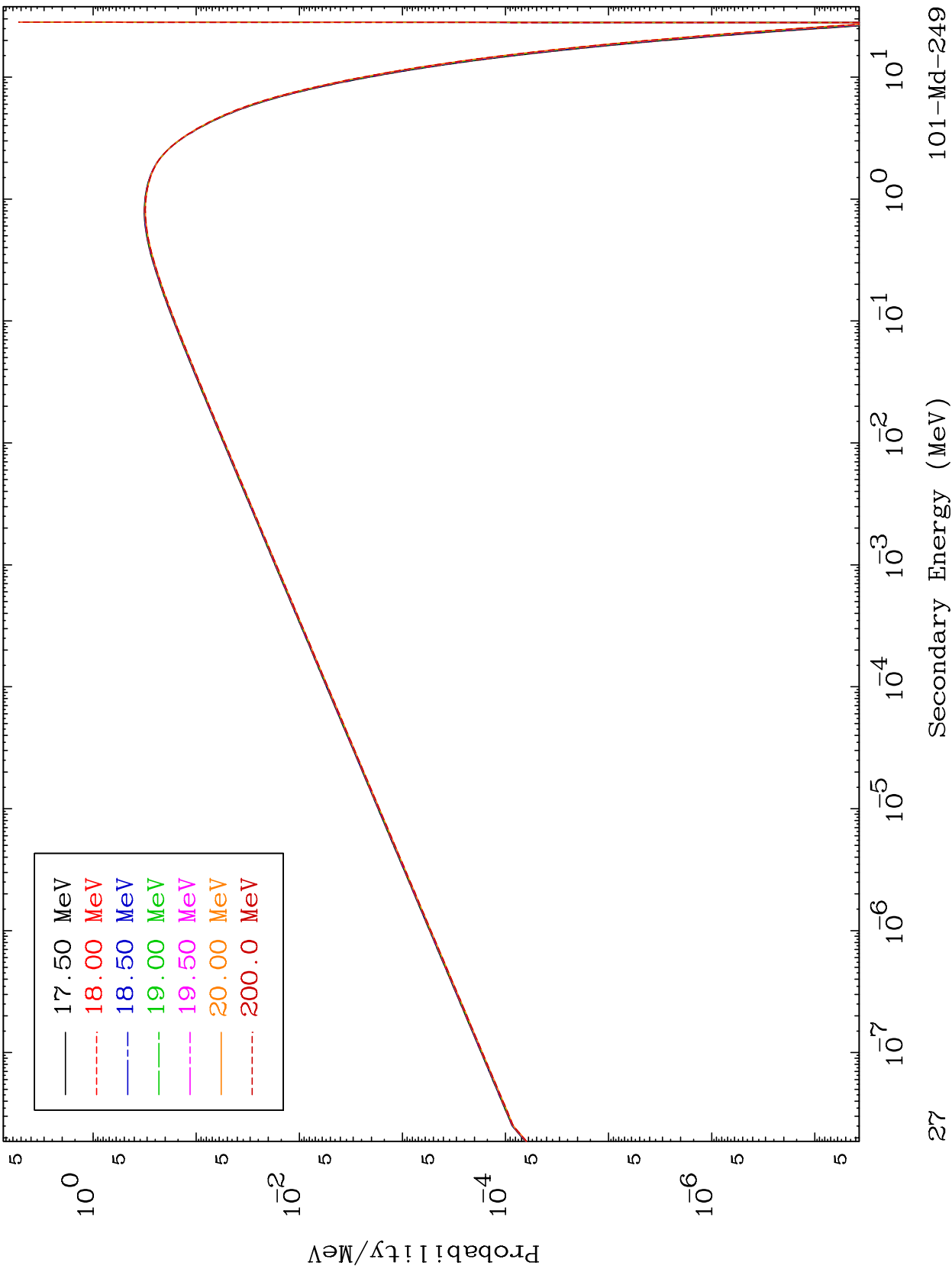
101-Md-249



MAT 9950

Fission
Energy Distribution

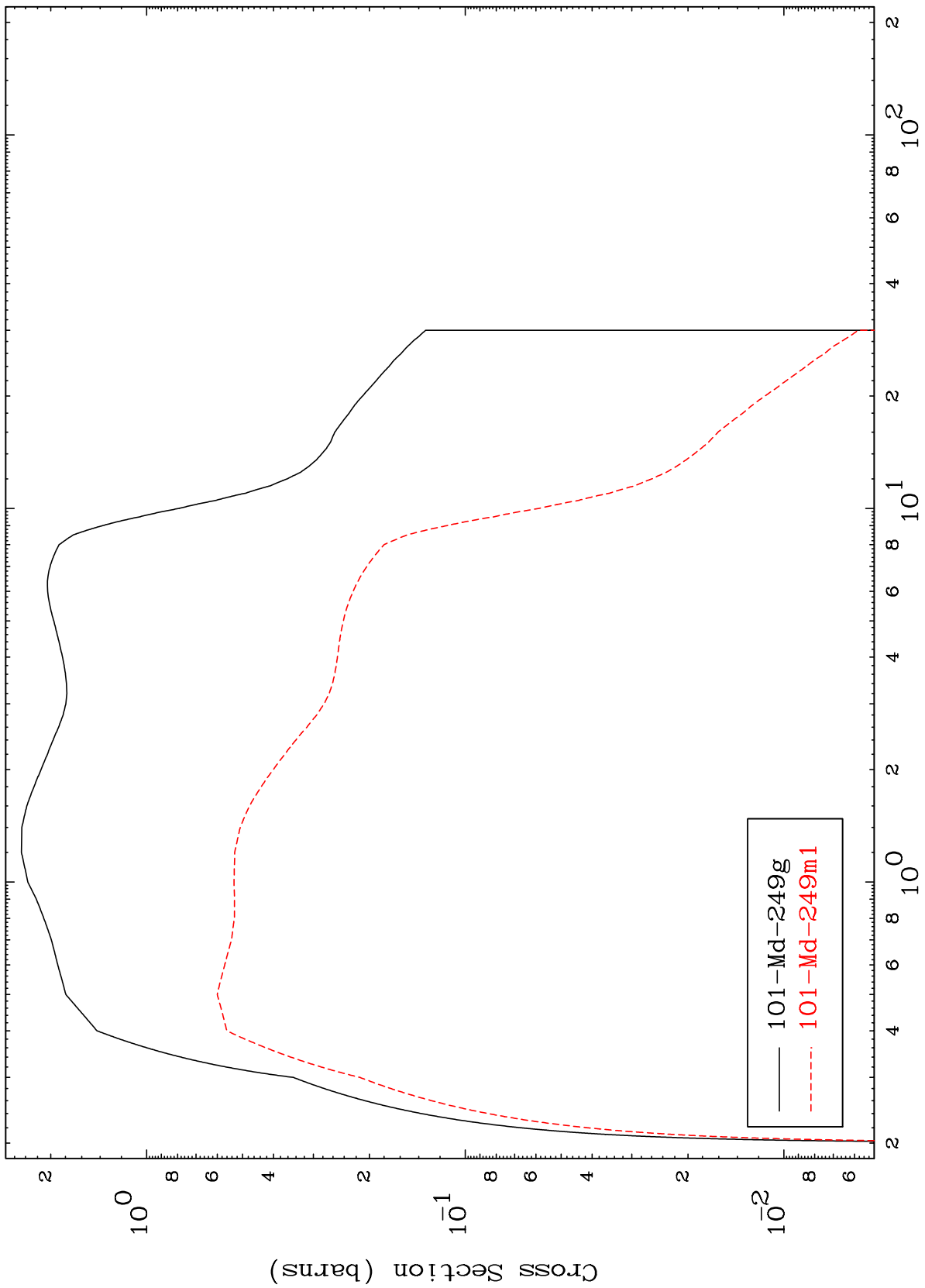
101-Md-249



MAT 9950

101-Md-249

Inelastic
Radionuclide Production Cross Section



— 101-Md-249g
- - - 101-Md-249m1

28

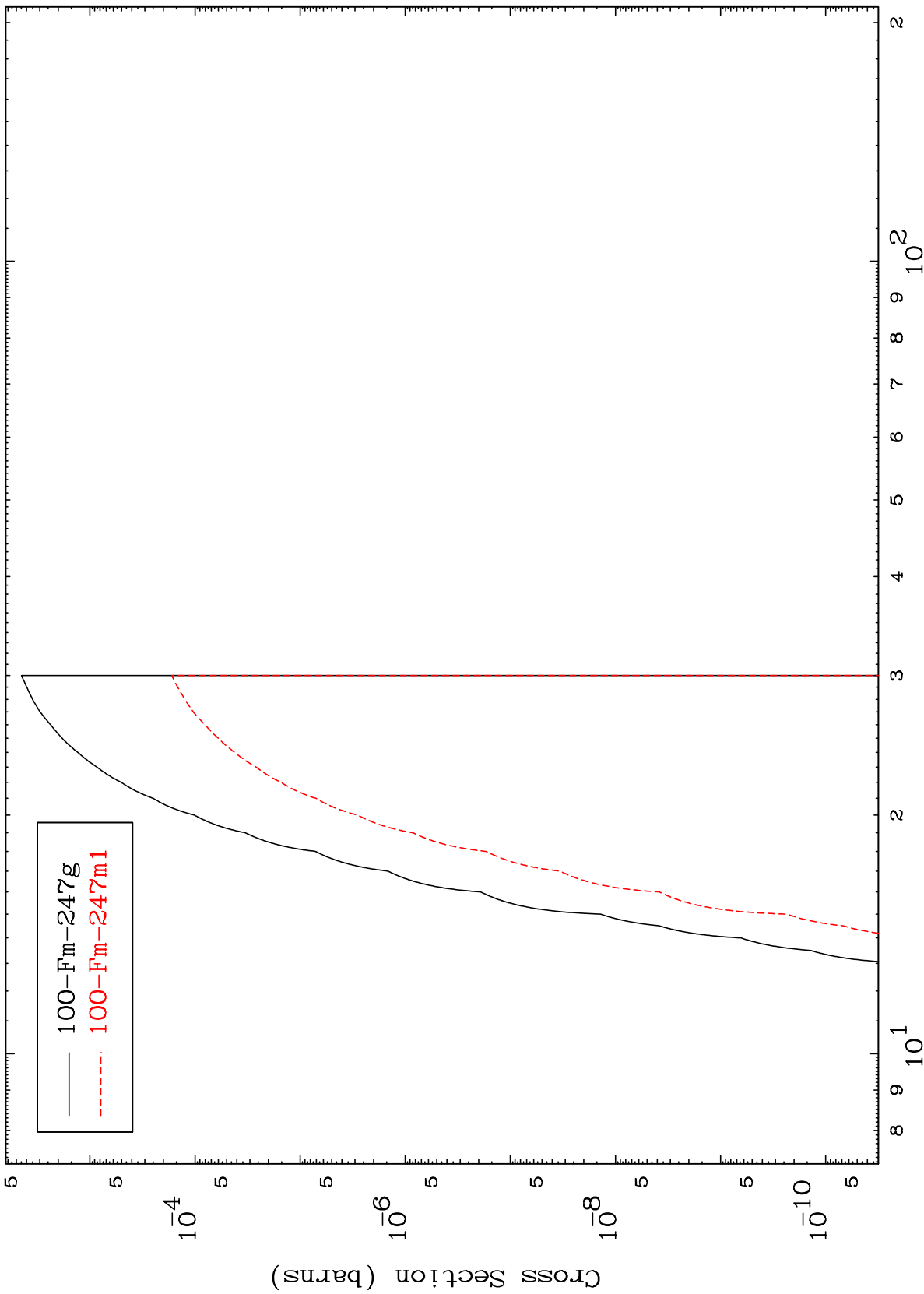
101-Md-249

MAT 9950

(n,n') d

101-Md-249

Radionuclide Production Cross Section



100-Fm-247g
100-Fm-247m1

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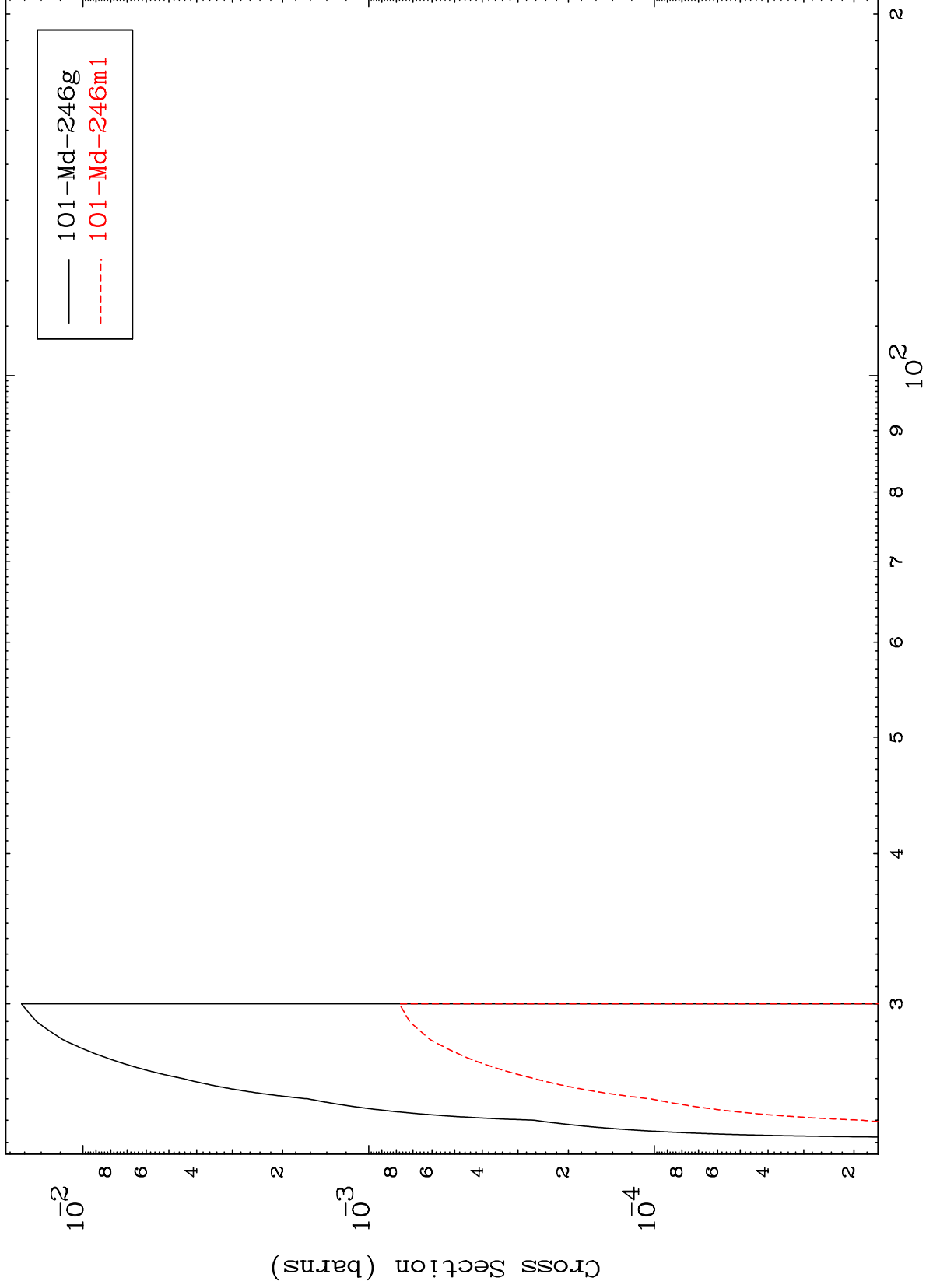
Incident Energy (MeV)

101-Md-249

MAT 9950

101-Md-249

(n,4n)
Radionuclide Production Cross Section



101-Md-249

Incident Energy (MeV)

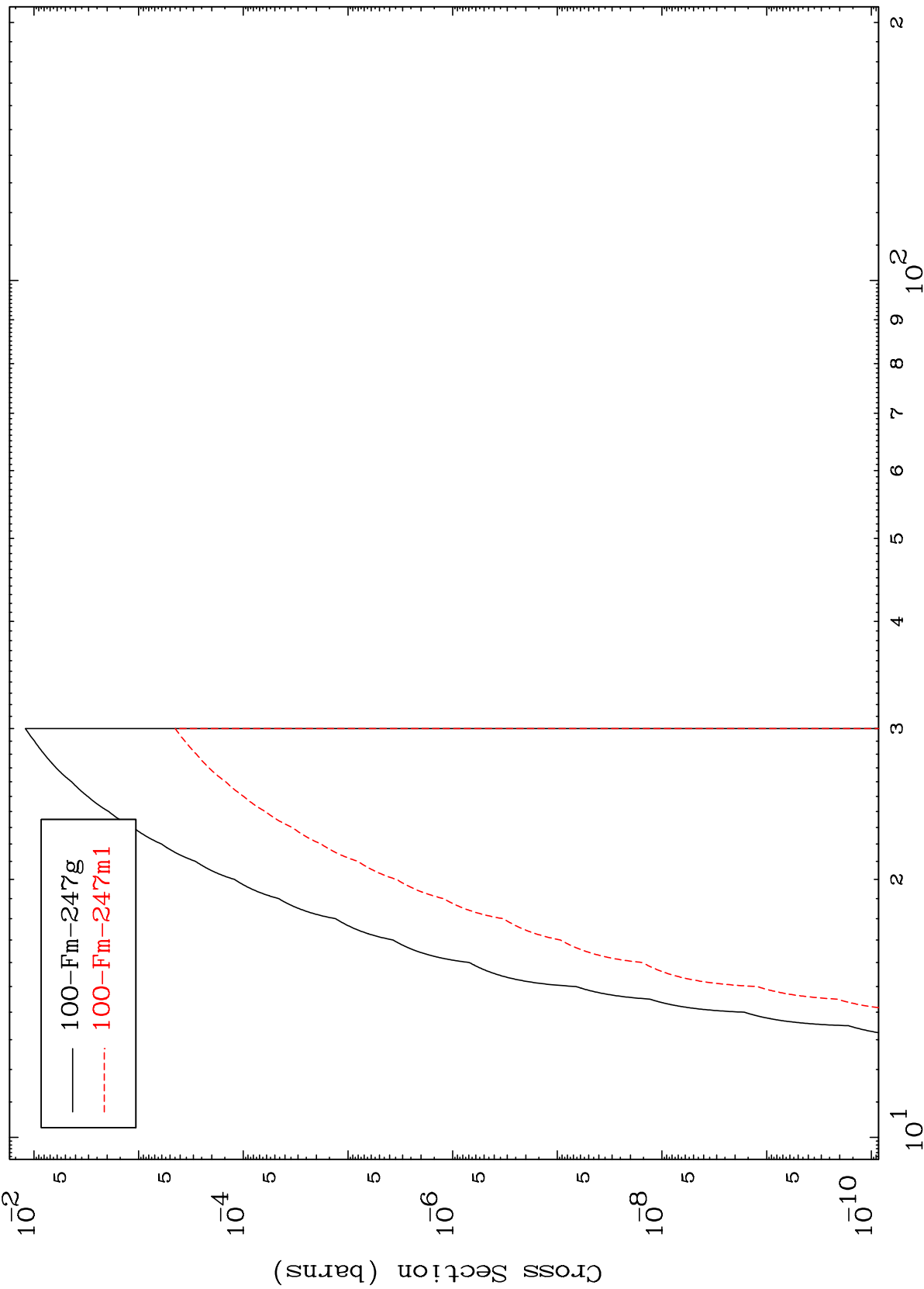
30

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(n,2n) p

101-Md-249

Radionuclide Production Cross Section



100-Fm-247g
100-Fm-247m1

Incident Energy (MeV)

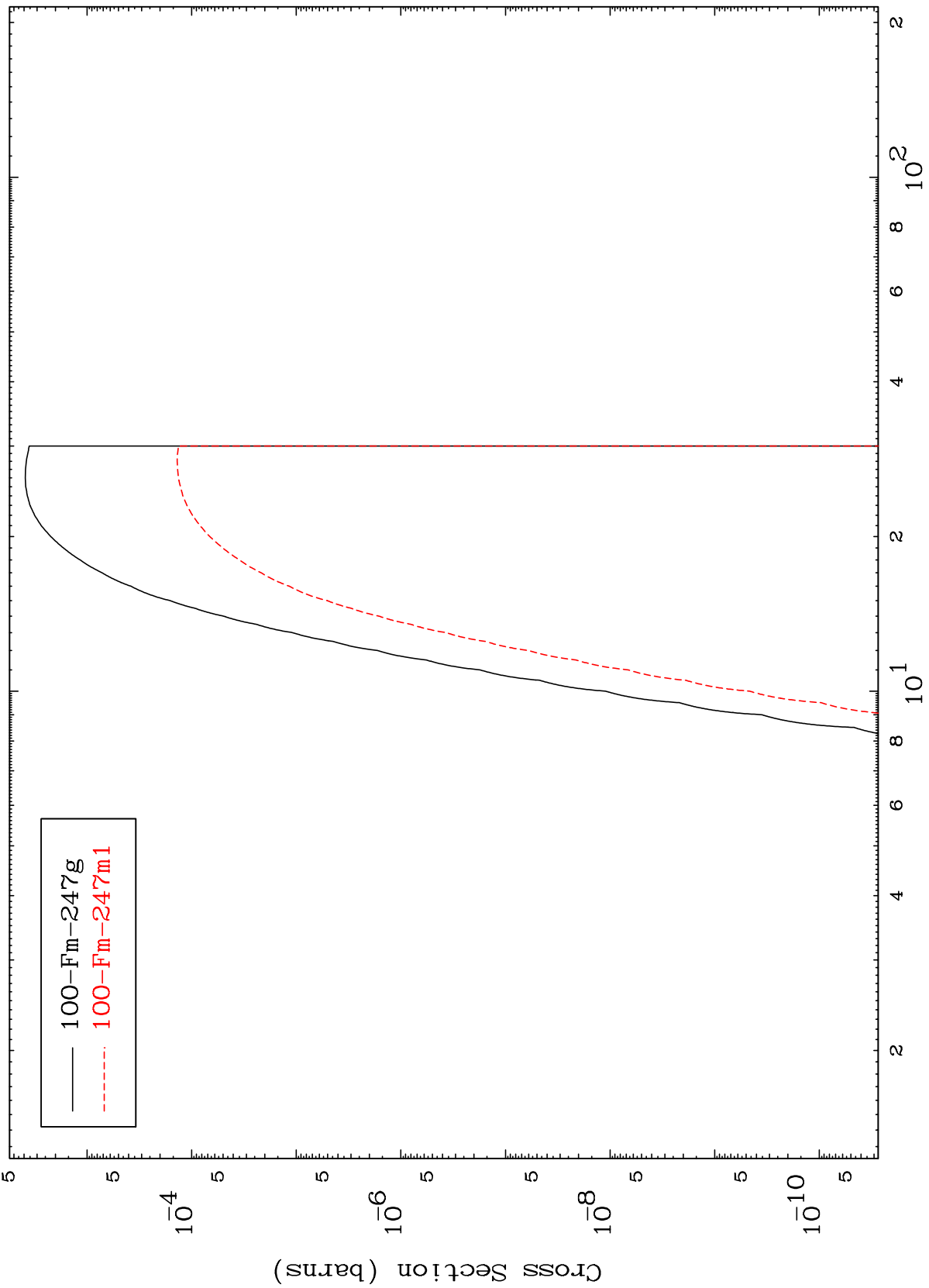
101-Md-249

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101-Md-249

(n, t)
Radionuclide Production Cross Section



101-Md-249

Incident Energy (MeV)

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