

Program EVALPLOT
(Version 2015-2)

by

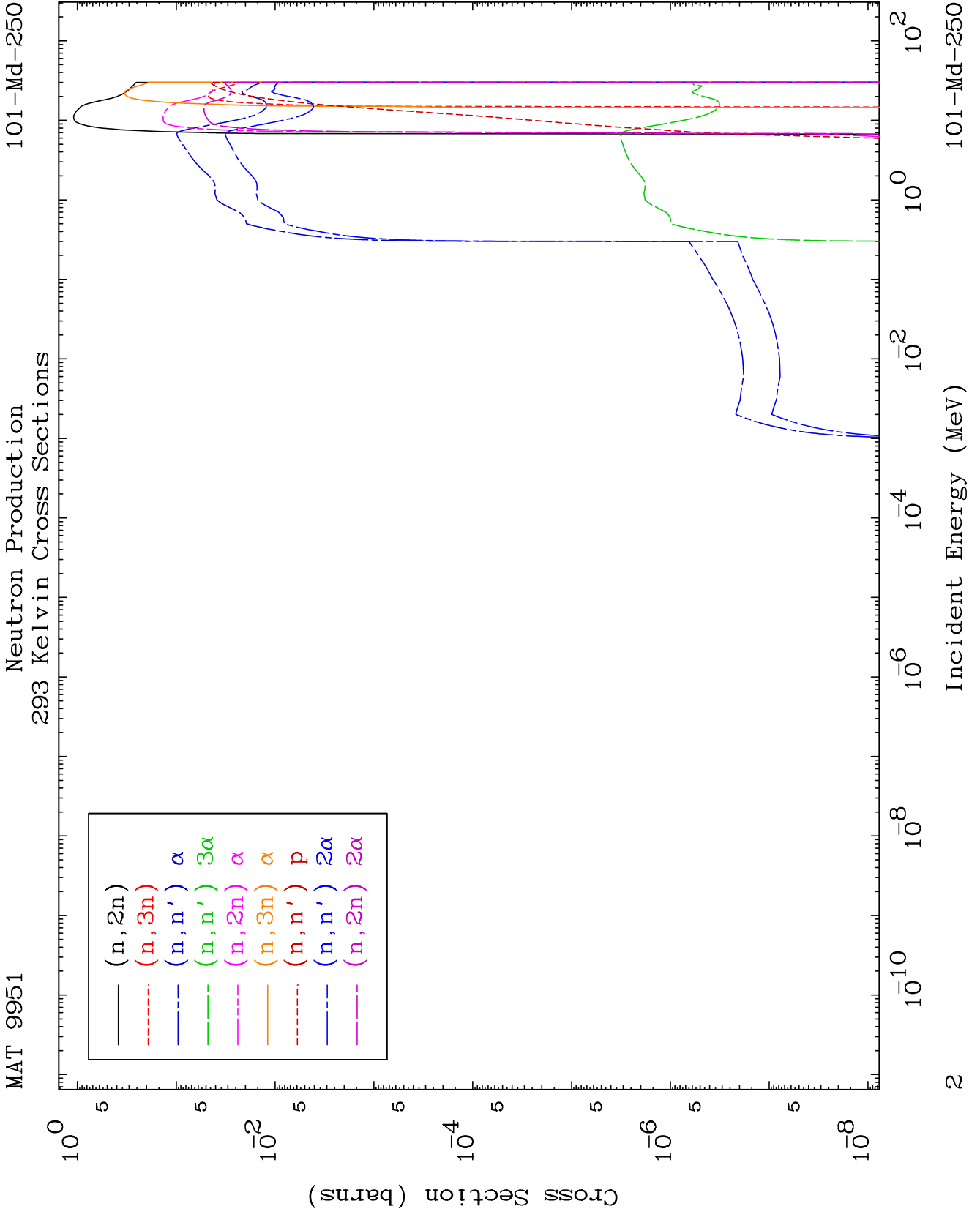
Dermott E. Cullen
(Present Contact Information)

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Livermore, CA 94550
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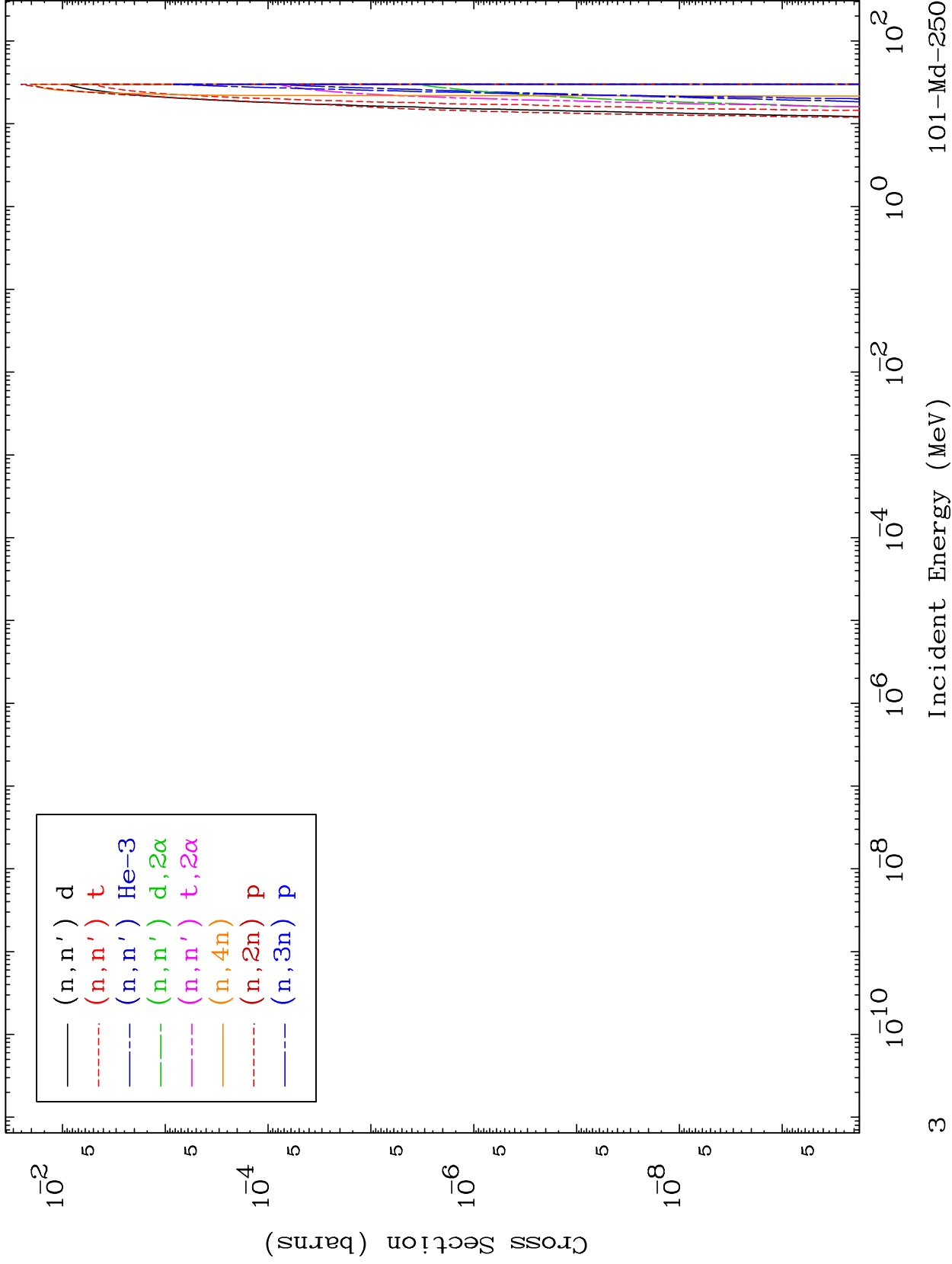
Press Mouse Button to Start



MAT 9951

Neutron Production
293 Kelvin Cross Sections

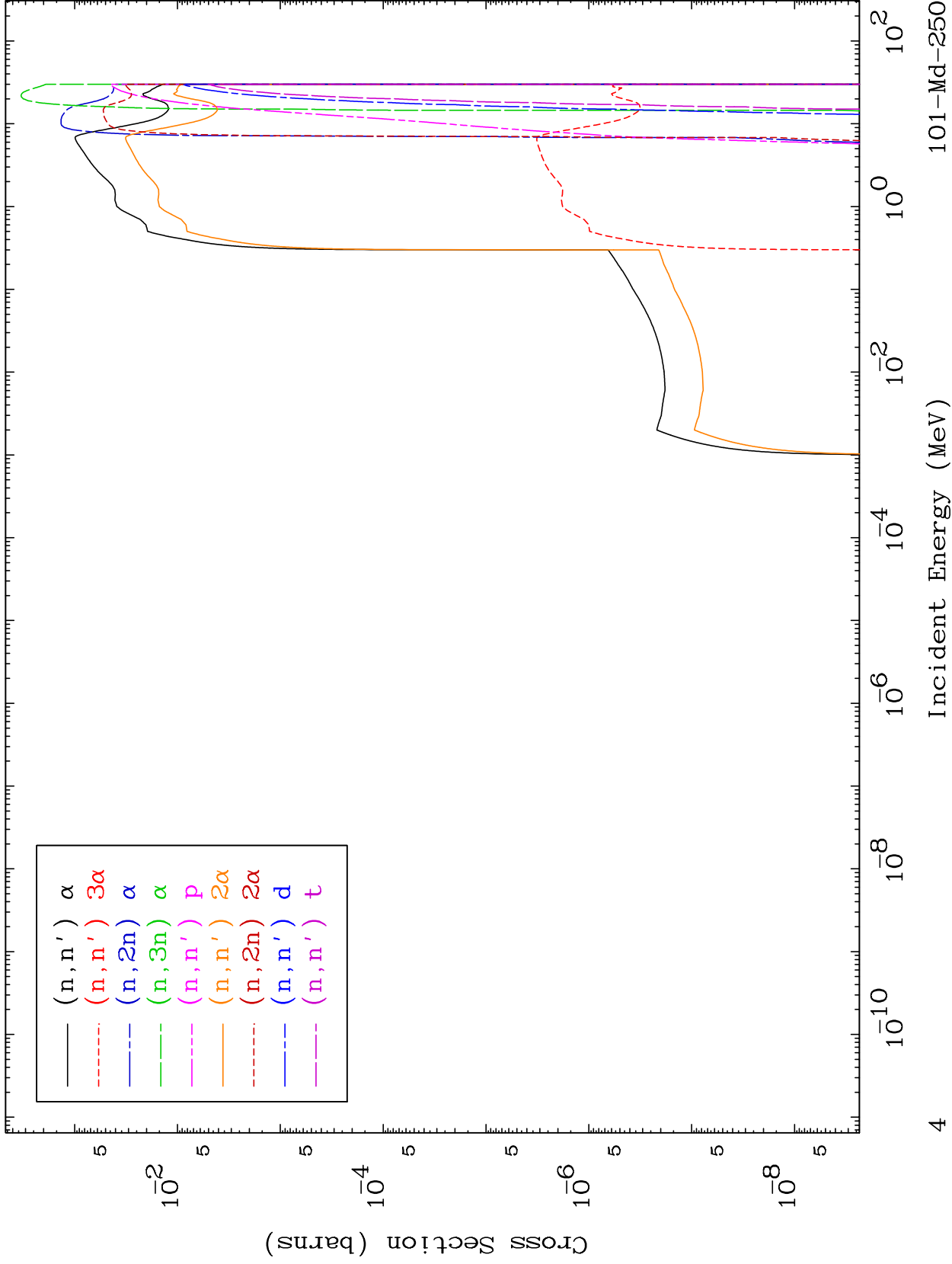
101-Md-250



MAT 9951

Charged Particle
293 Kelvin Cross Sections

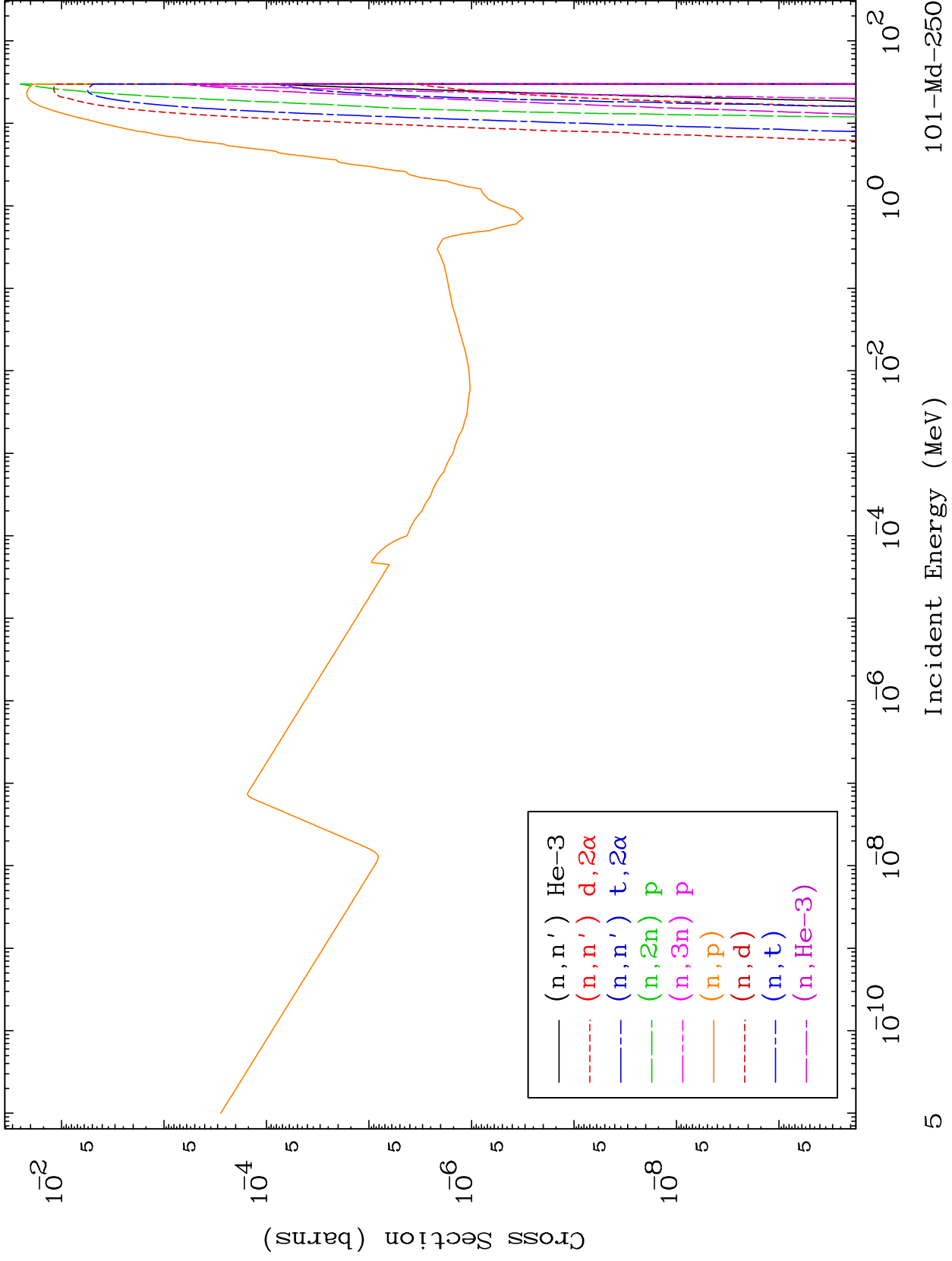
101-Md-250

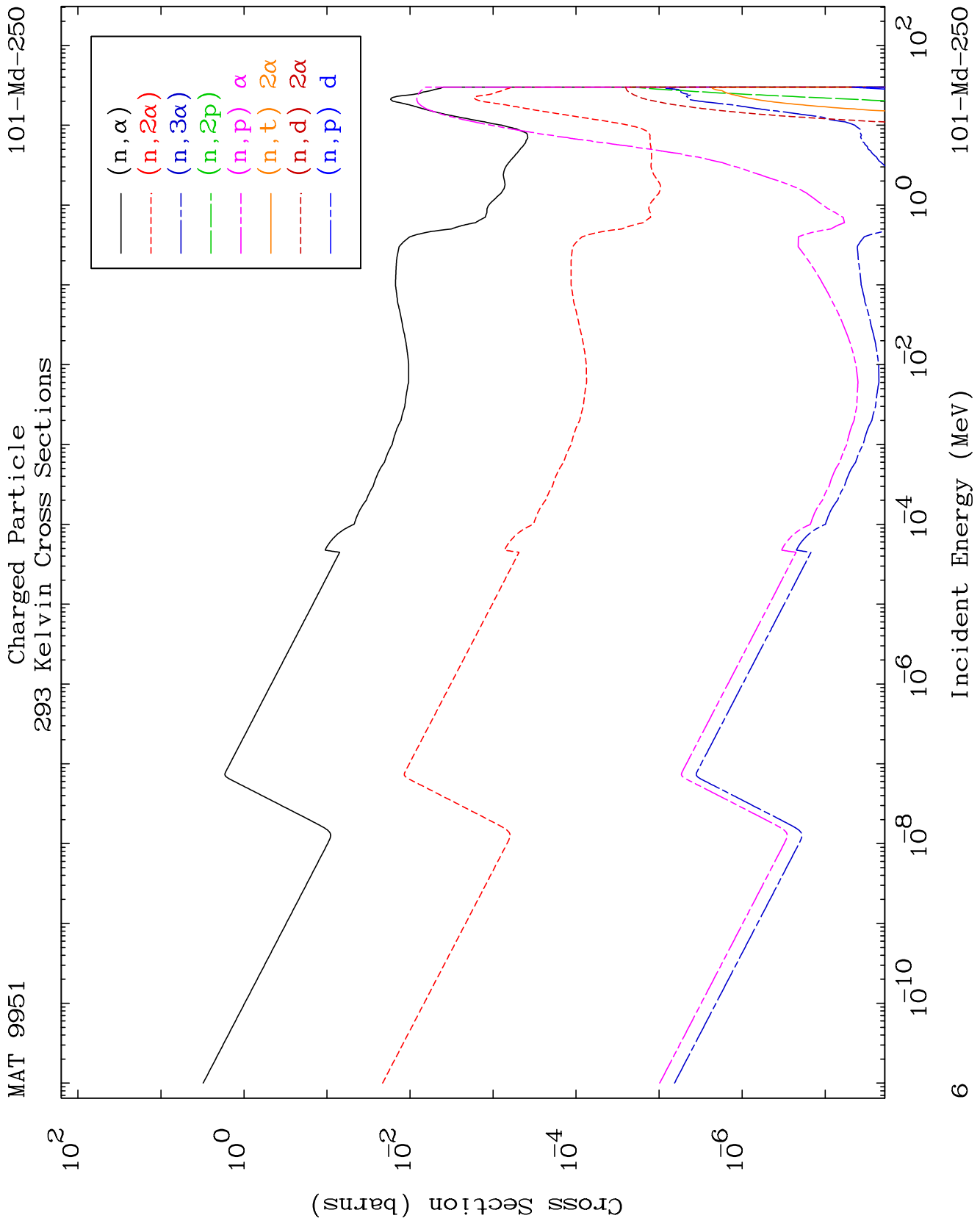


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Charged Particle
293 Kelvin Cross Sections

101-Md-250

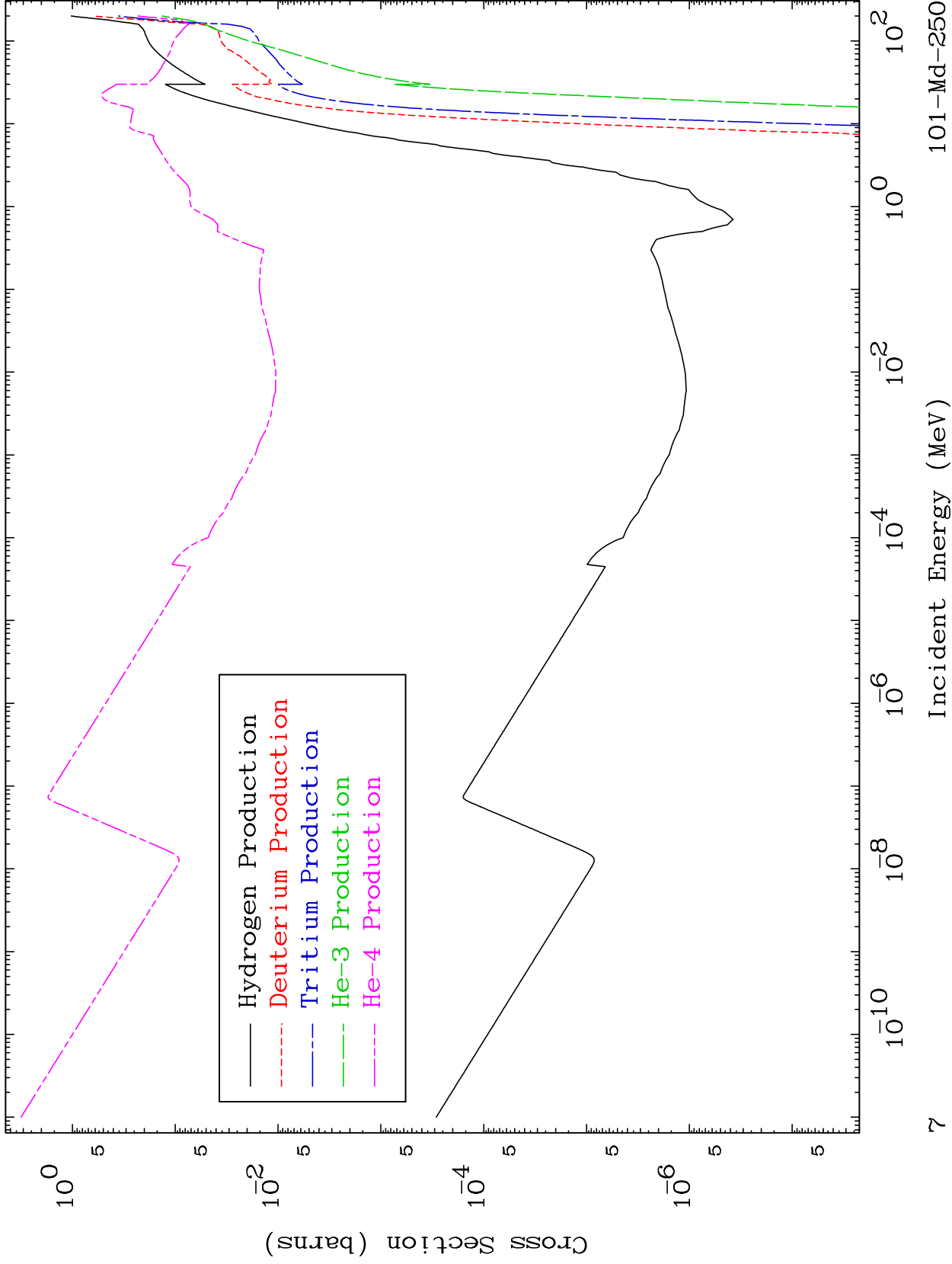




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Particle Production
293 Kelvin Cross Sections

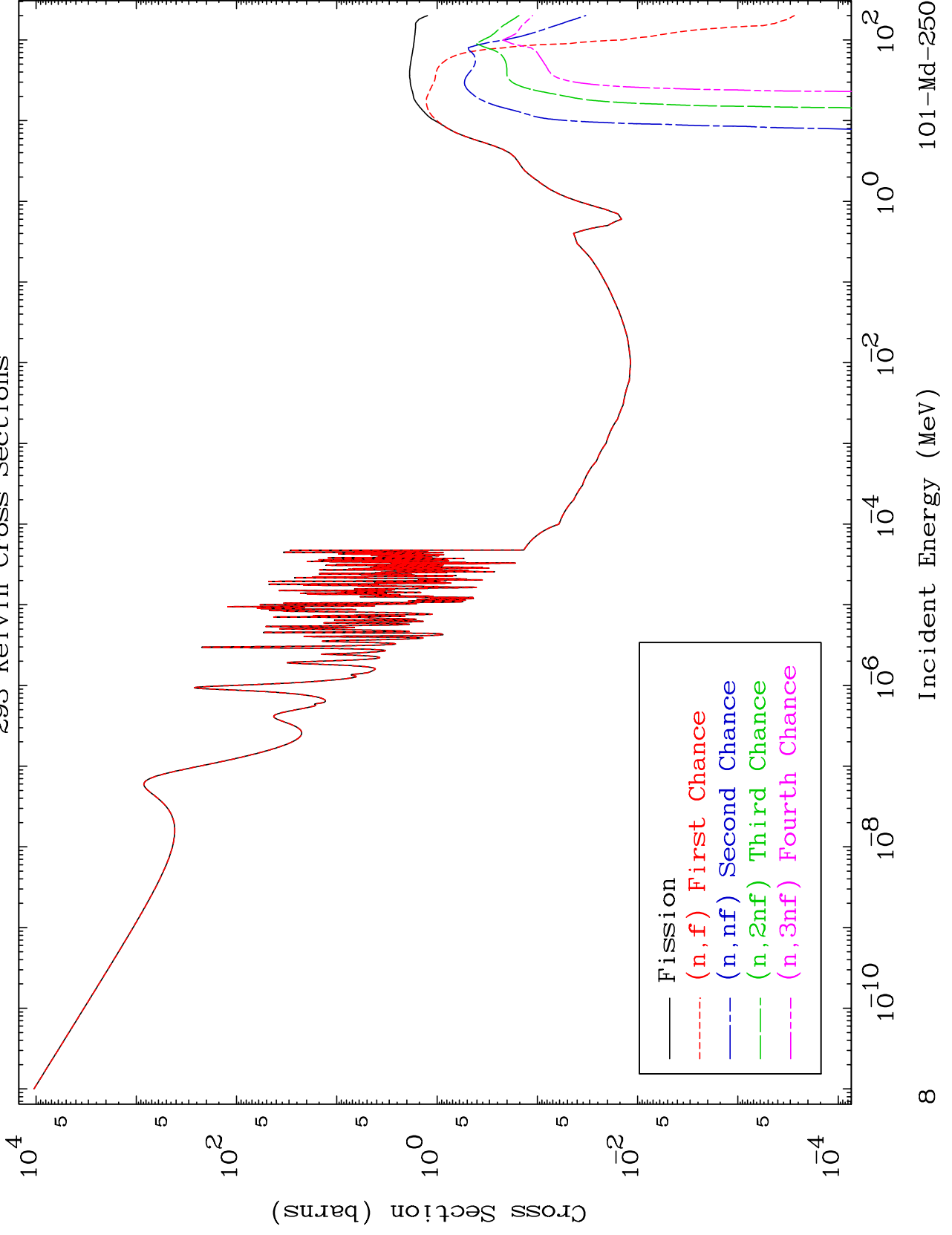
101-Md-250



MAT 9951

Fission
293 Kelvin Cross Sections

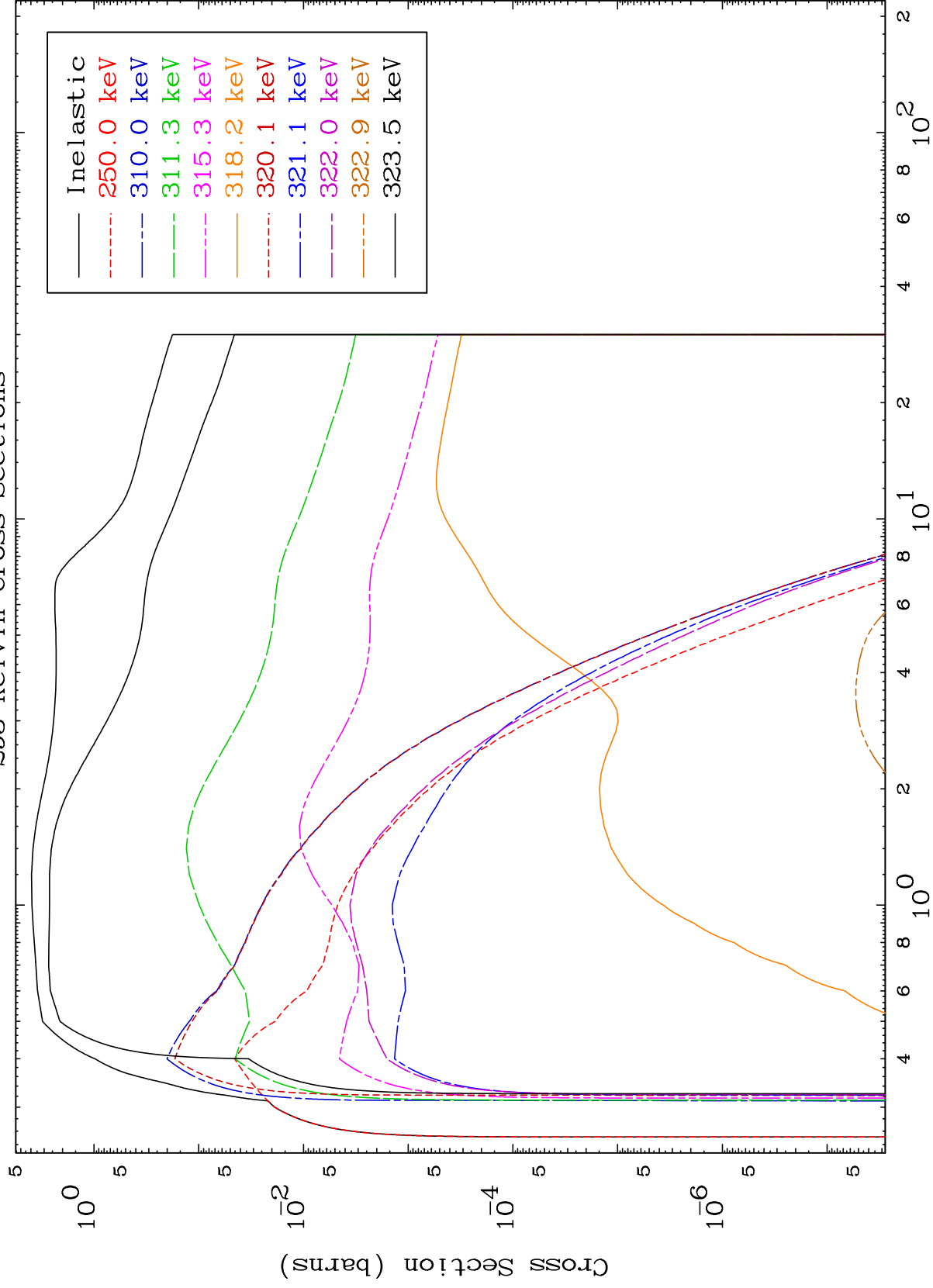
101-Md-250



MAT 9951

(n,n') Level
293 Kelvin Cross Sections

101-Md-250



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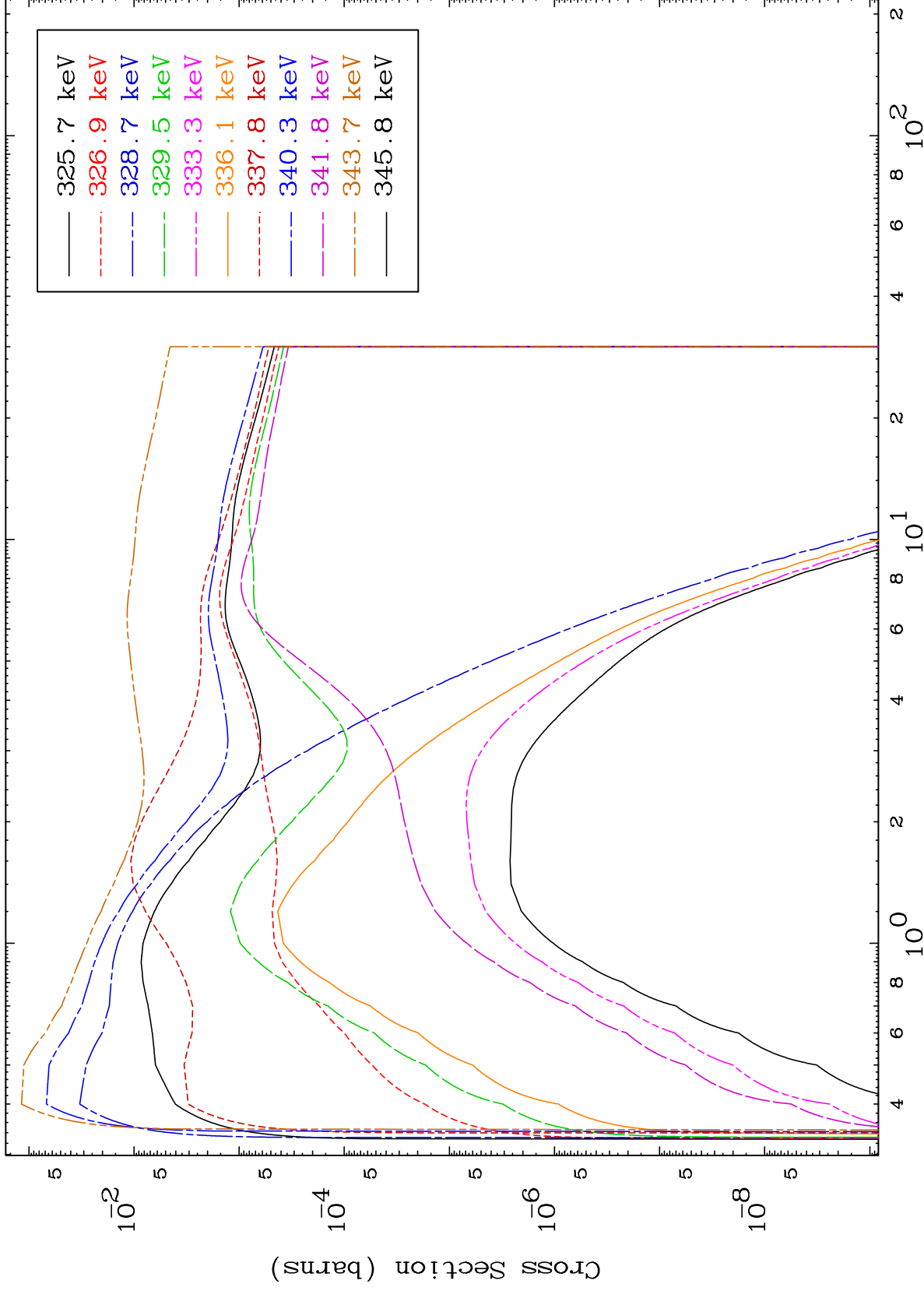
Incident Energy (MeV)

101-Md-250

MAT 9951

(n,n') Level
293 Kelvin Cross Sections

101-Md-250



10

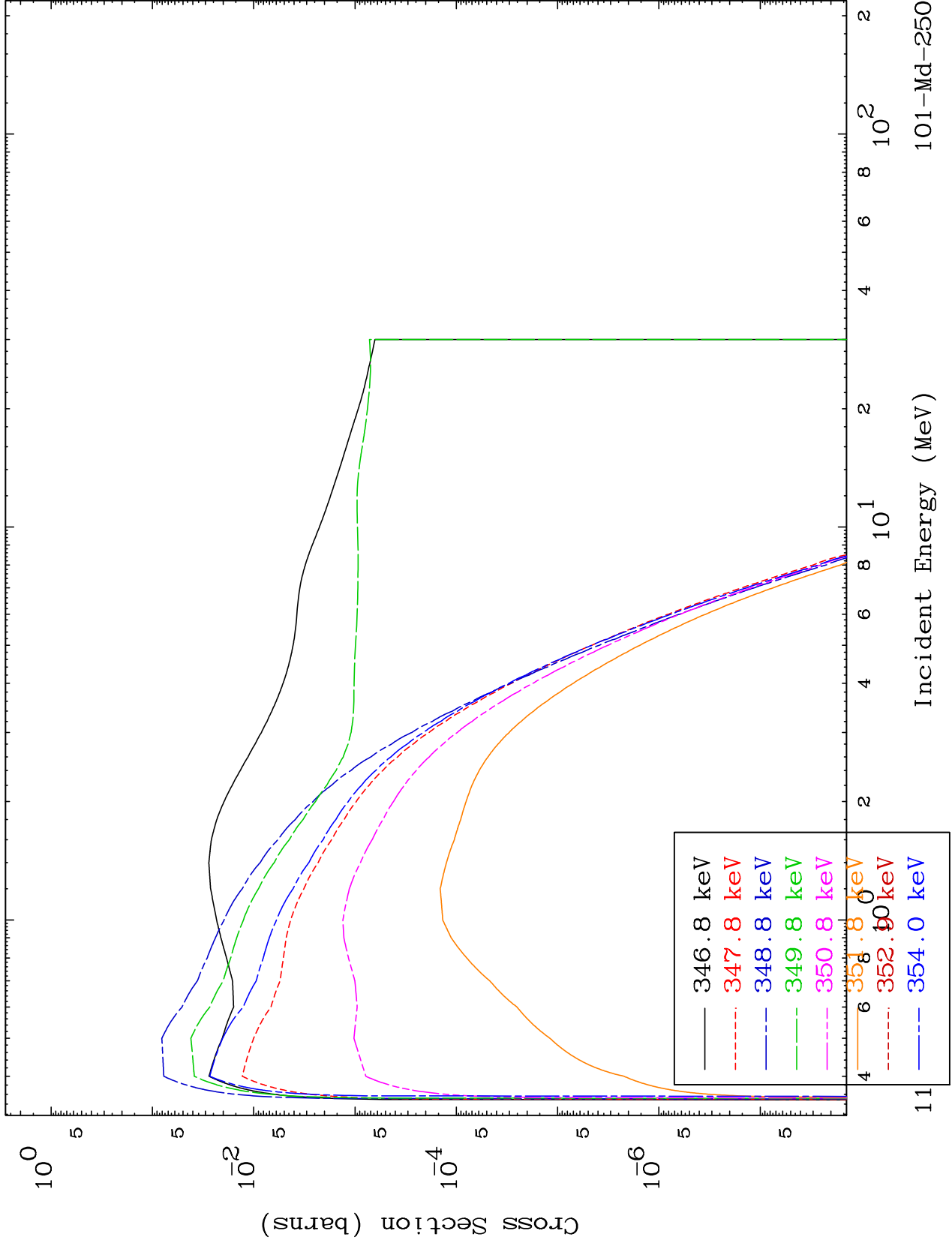
Incident Energy (MeV)

101-Md-250

MAT 9951

(n,n') Level
293 Kelvin Cross Sections

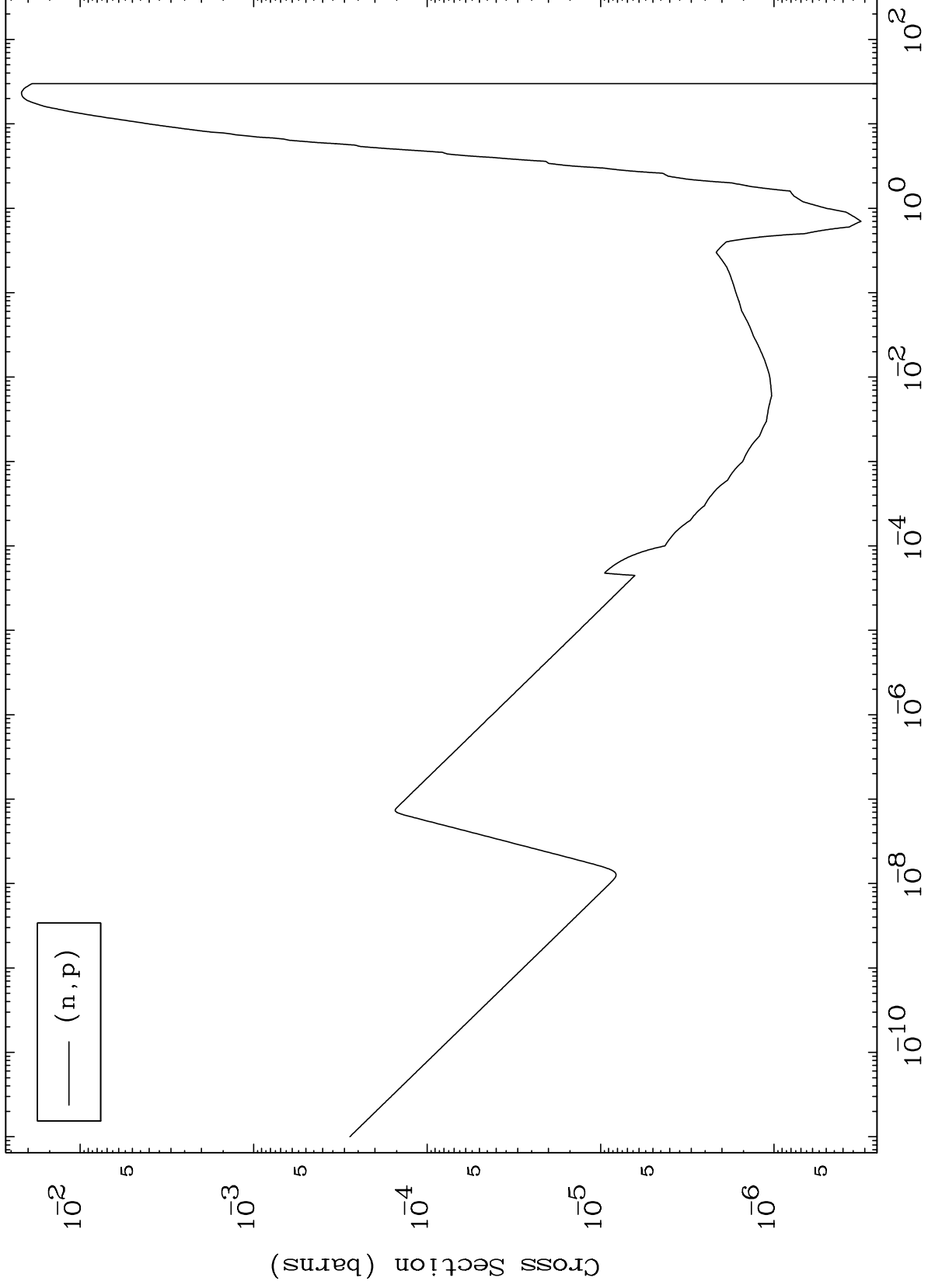
101-Md-250



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(n,p) Levels
293 Kelvin Cross Sections

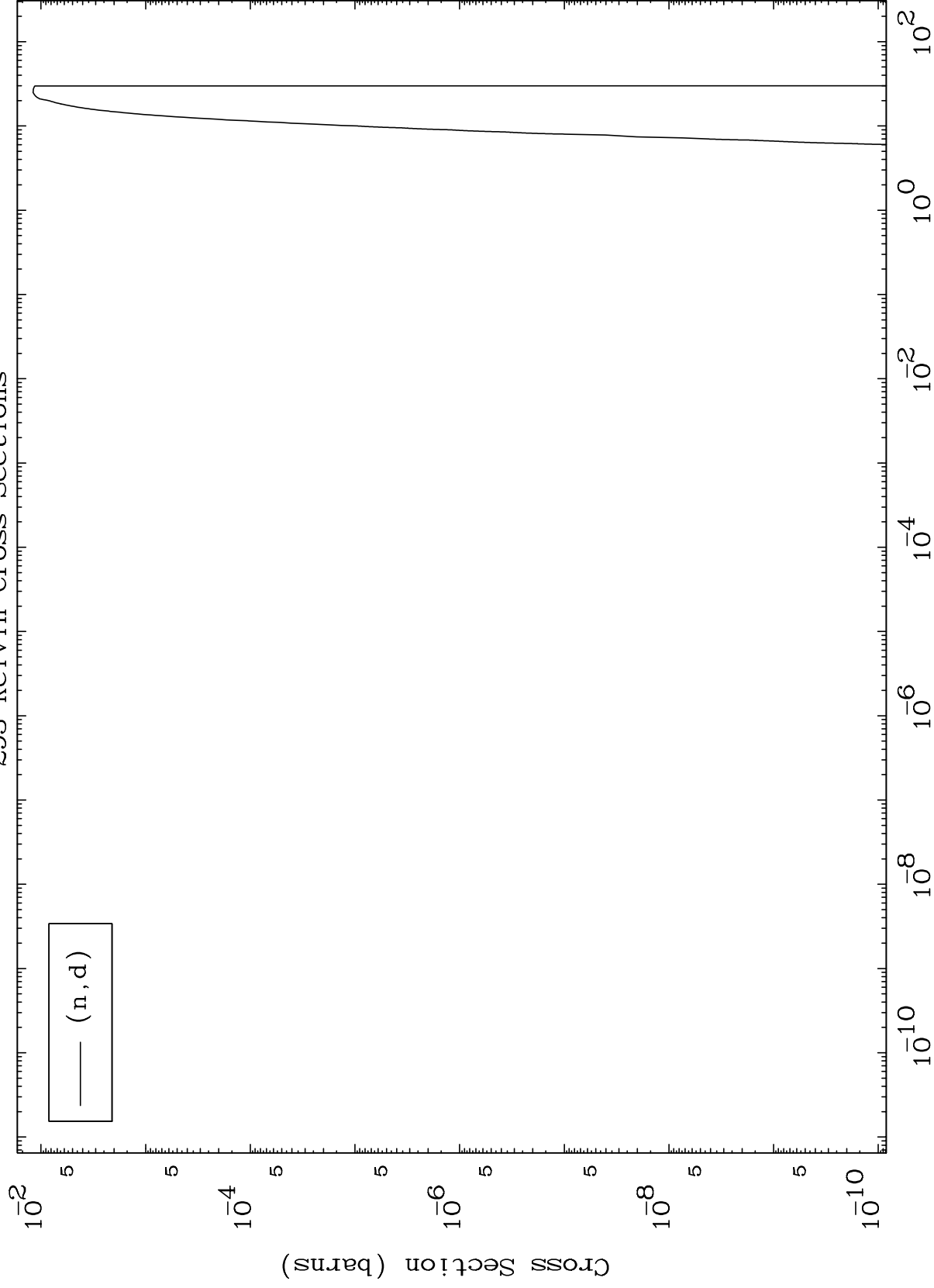
101-Md-250



MAT 9951

(n,d) Levels
293 Kelvin Cross Sections

101-Md-250



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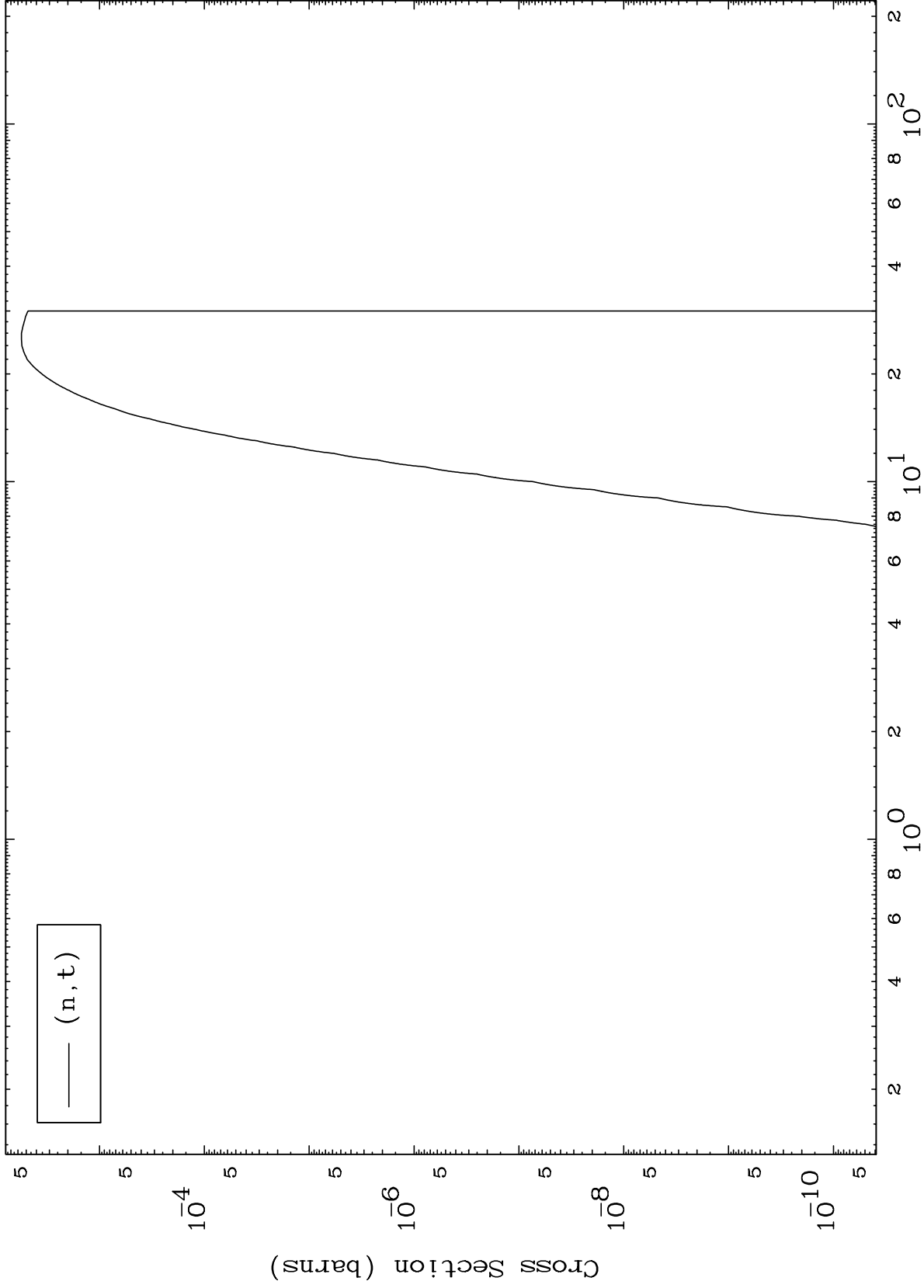
Incident Energy (MeV)

101-Md-250

MAT 9951

(n,t) Levels
293 Kelvin Cross Sections

101-Md-250



14

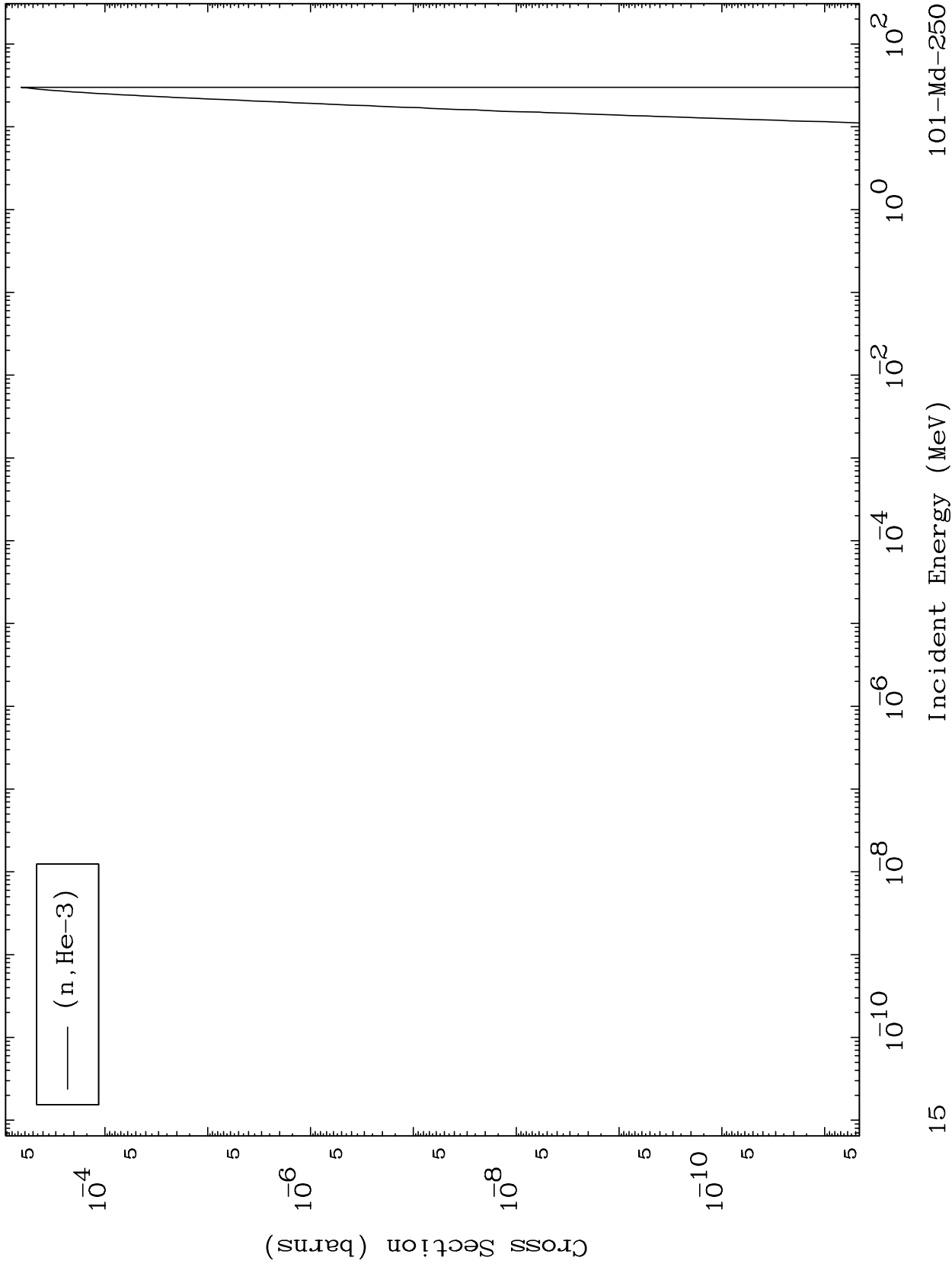
Incident Energy (MeV)

101-Md-250

MAT 9951

(n,He3) Levels
293 Kelvin Cross Sections

101-Md-250

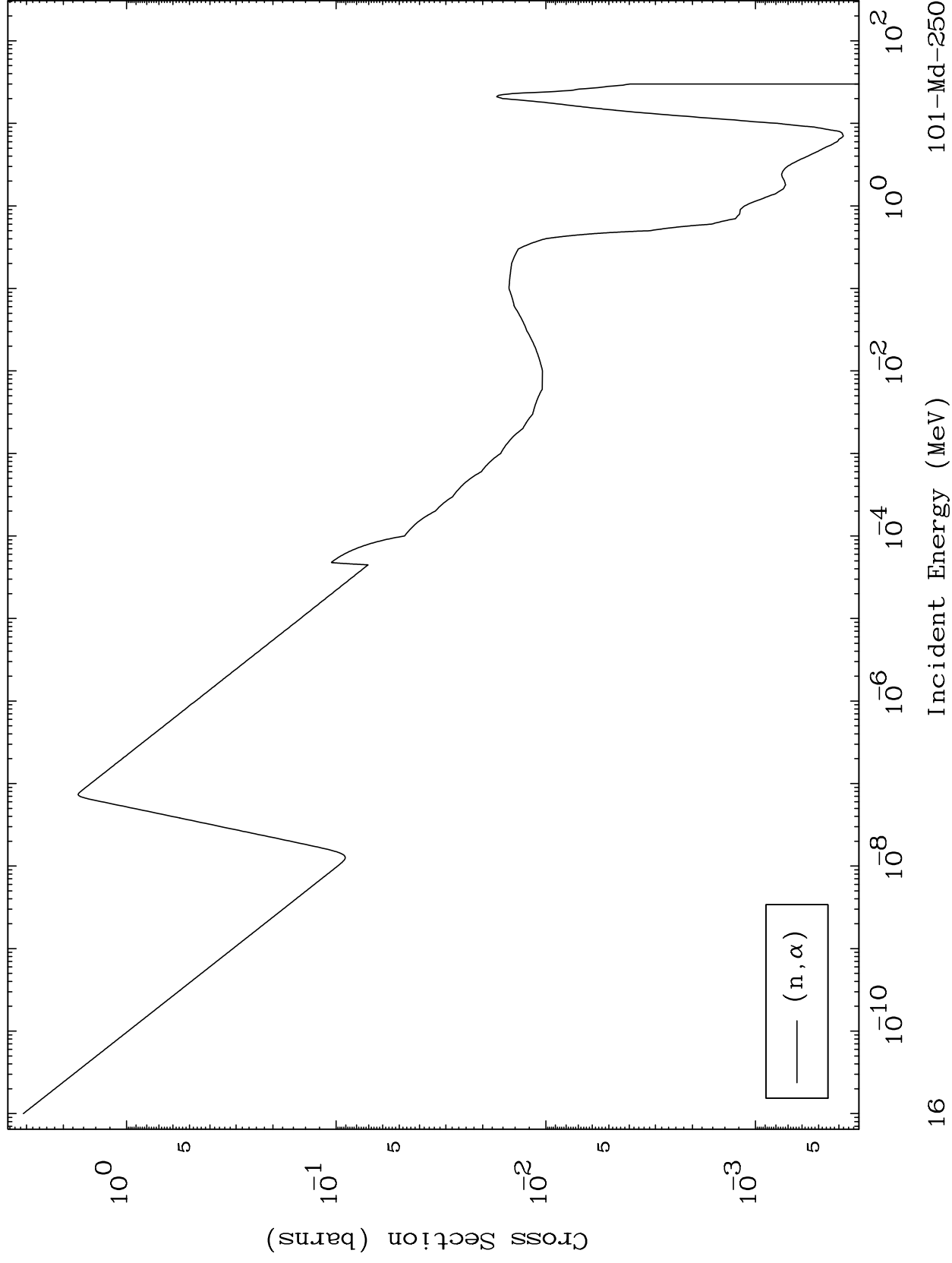


15

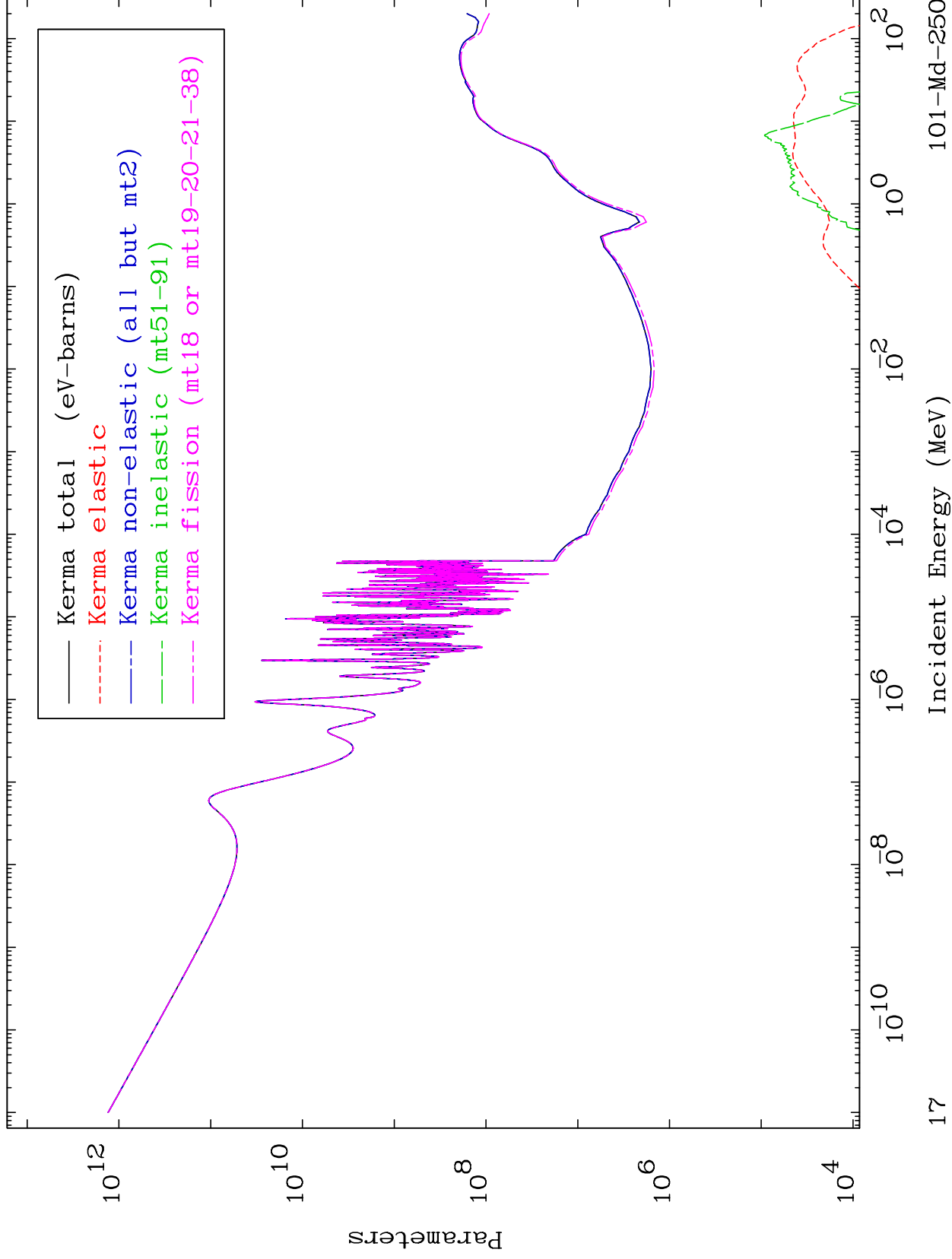
MAT 9951

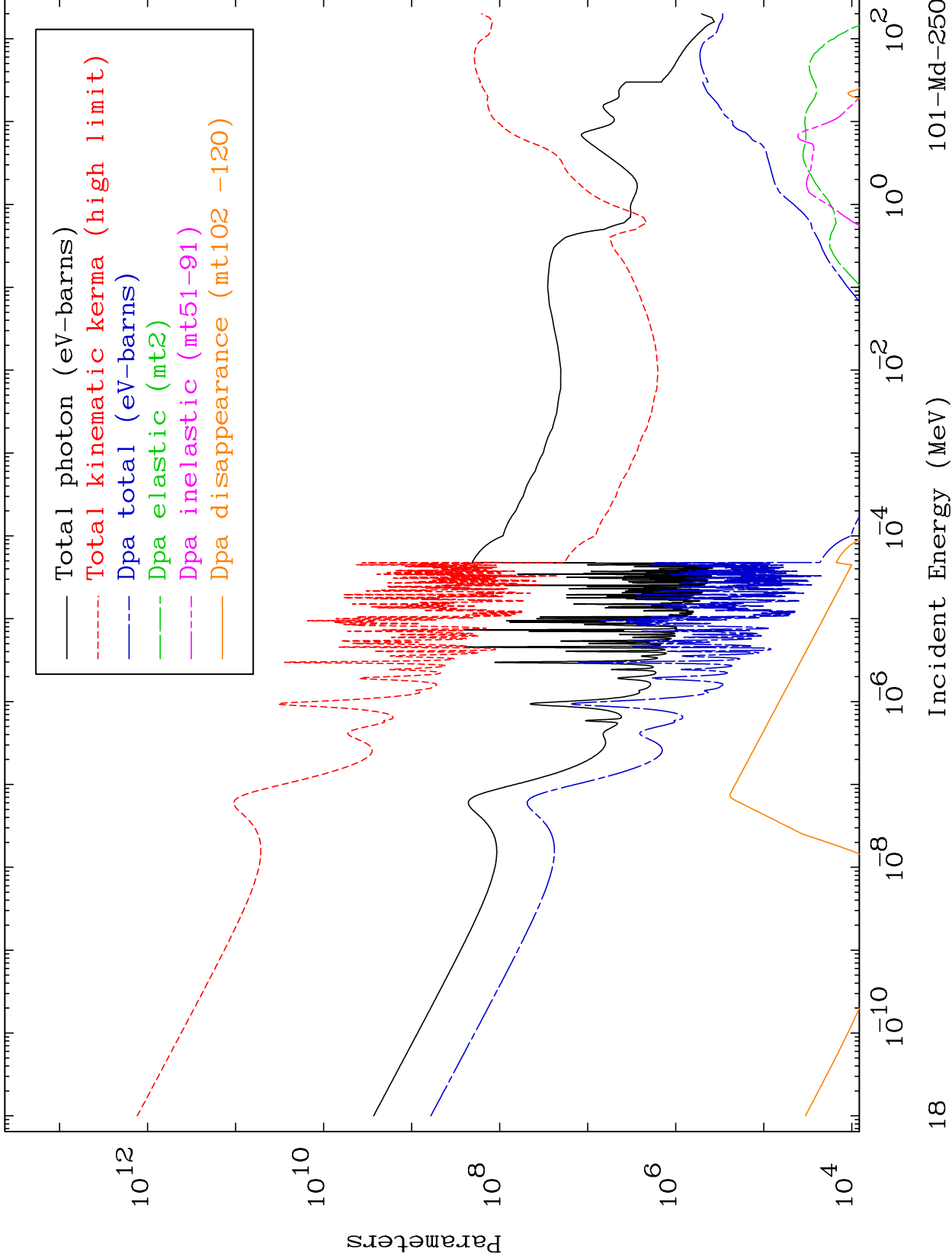
(n,α) Levels
293 Kelvin Cross Sections

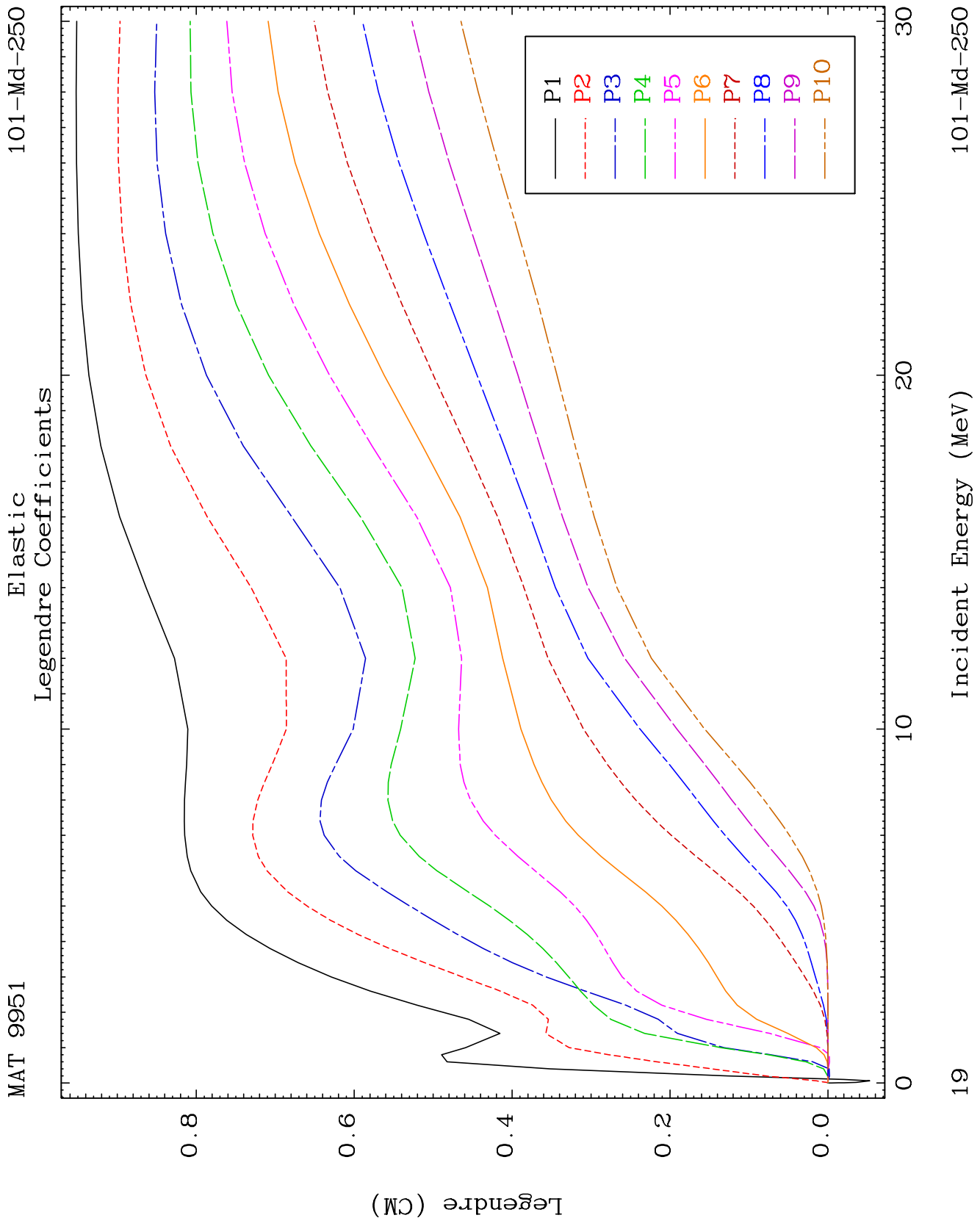
101-Md-250

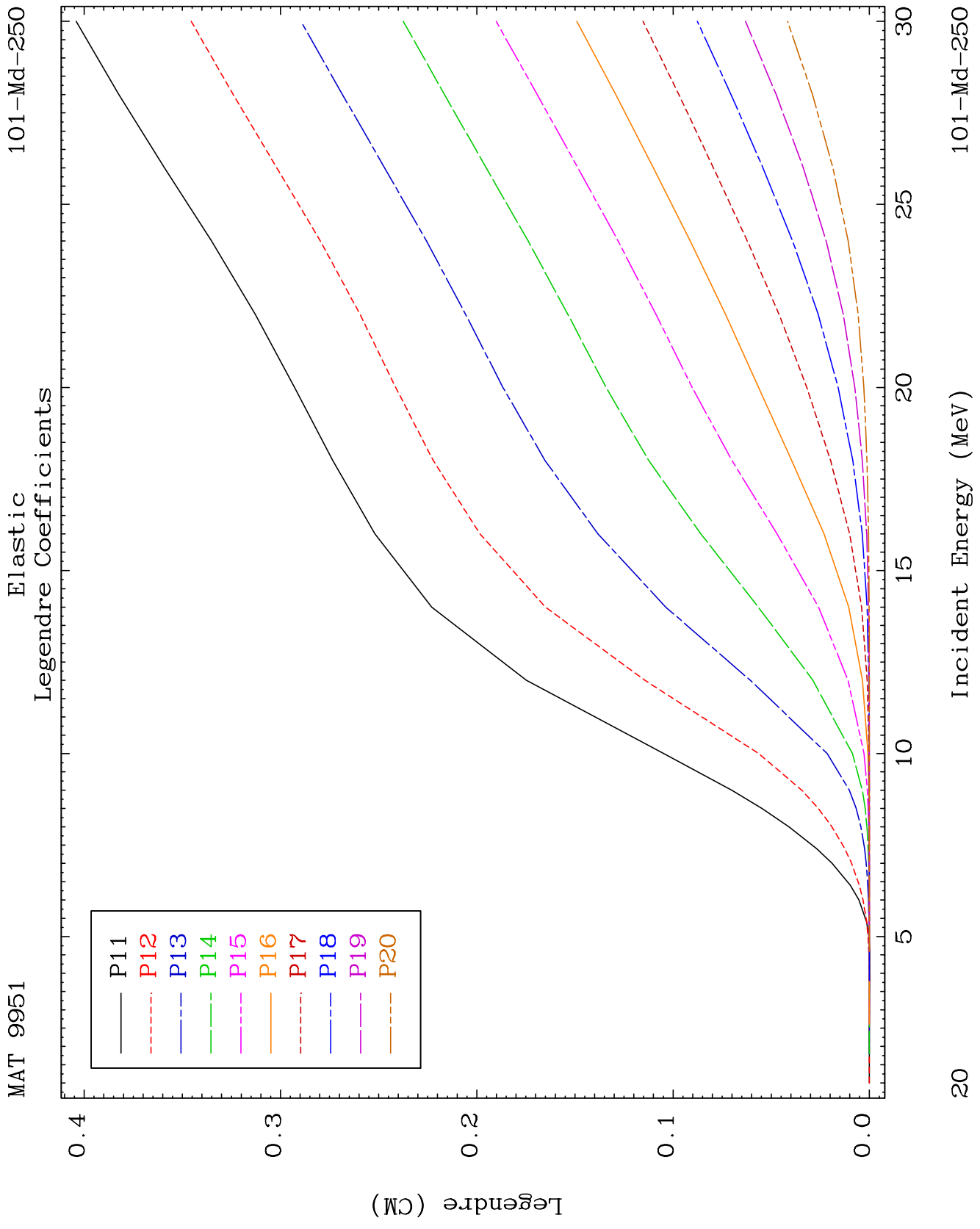


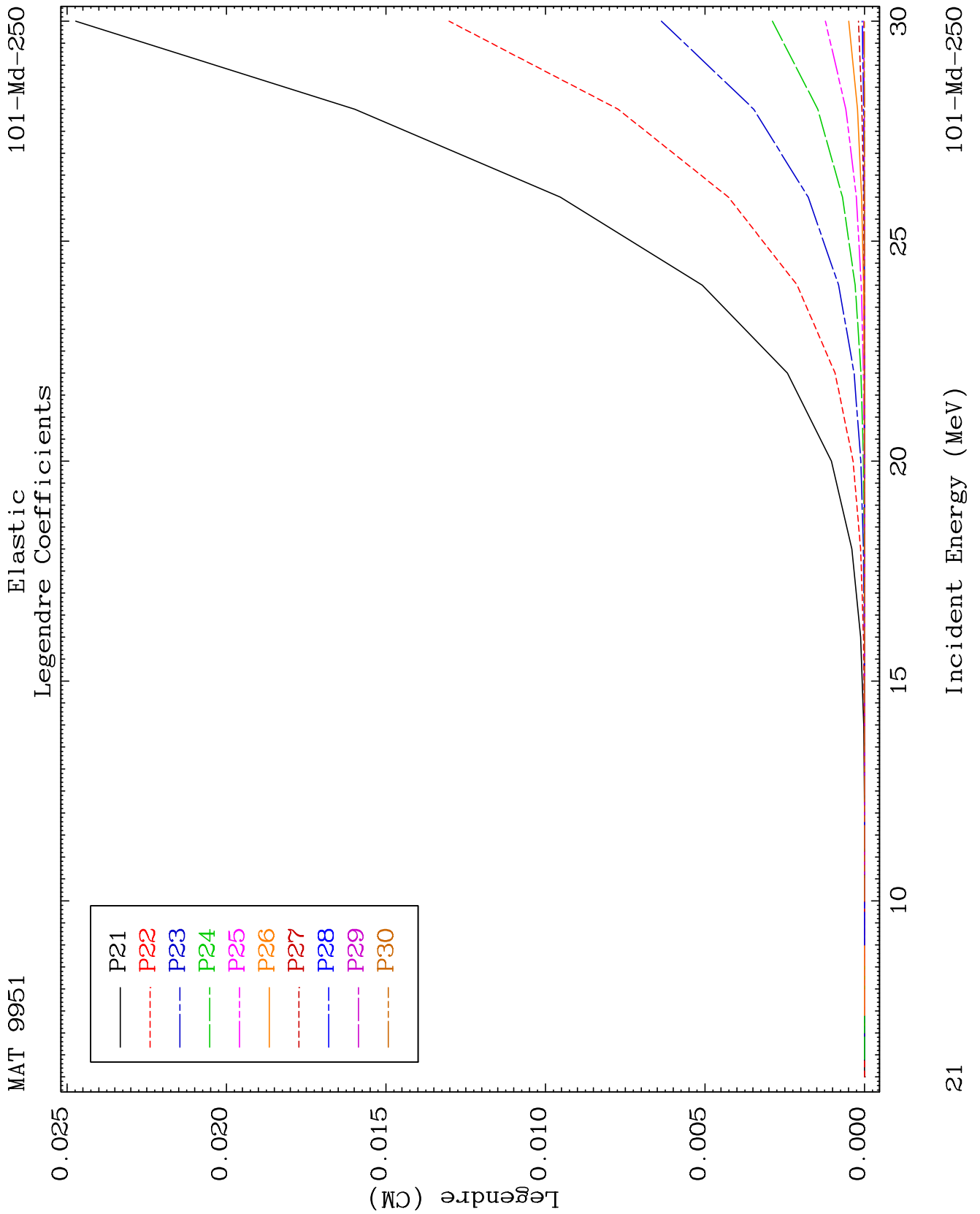
16







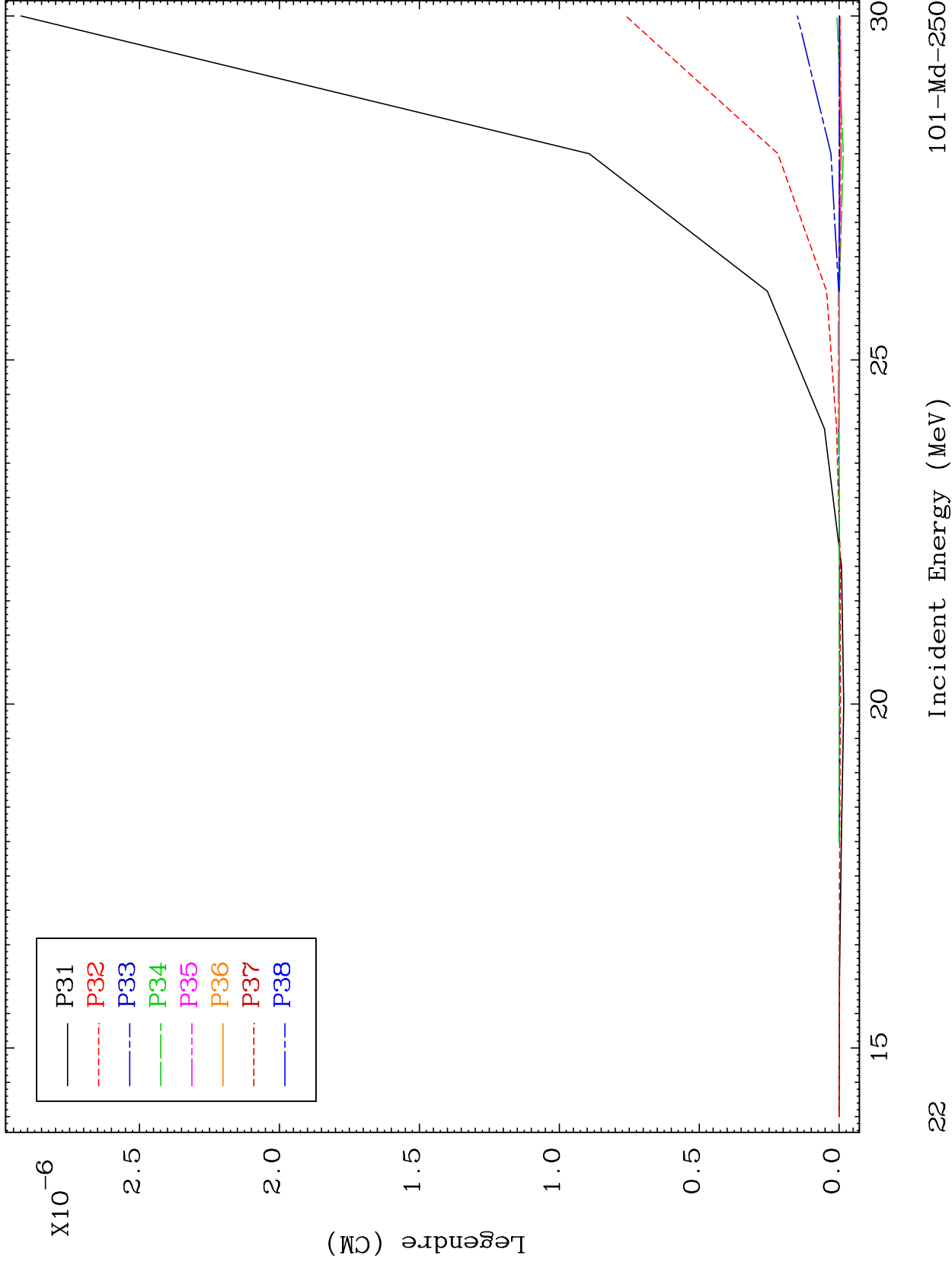




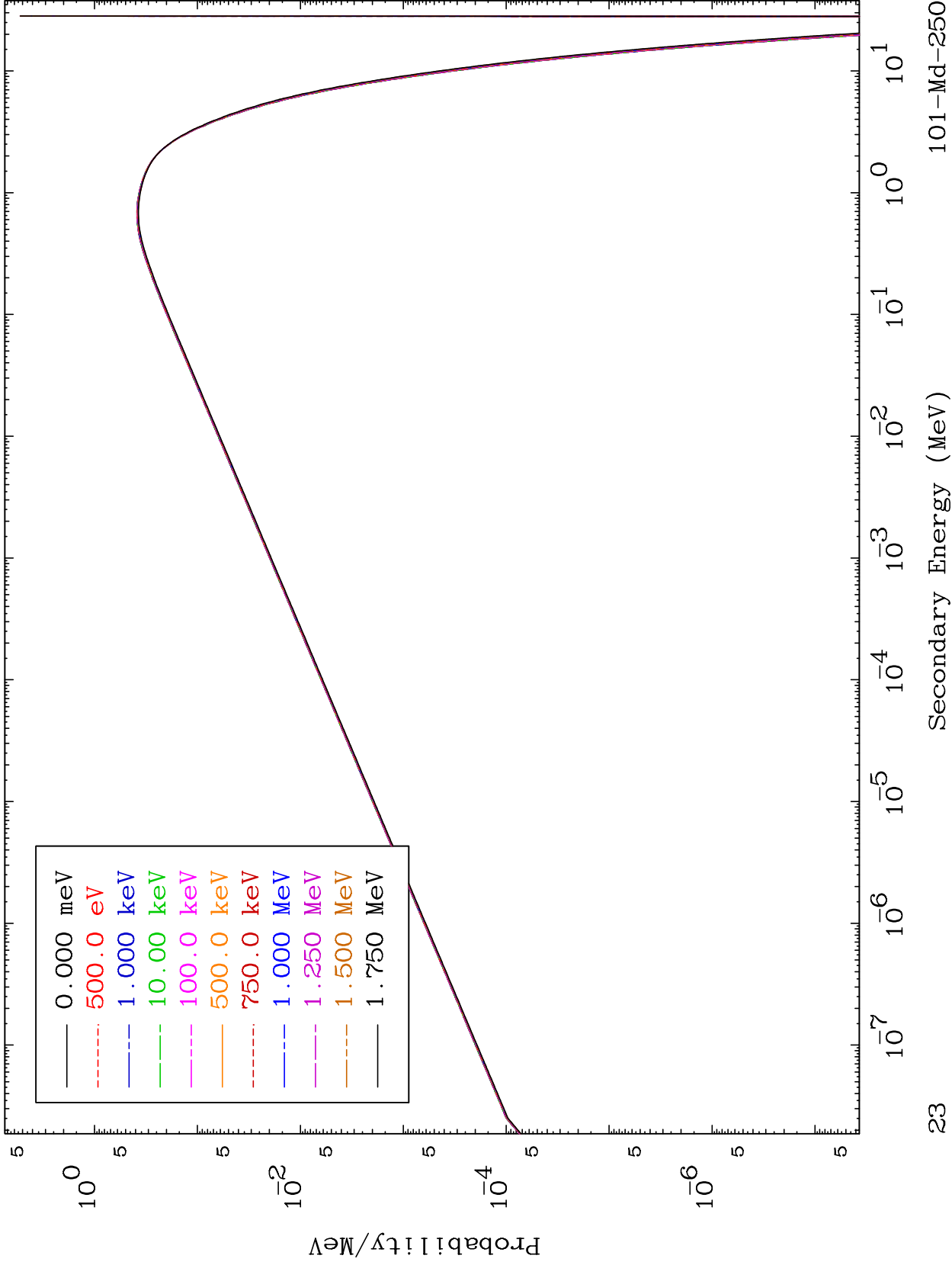
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Elastic
Legendre Coefficients

101-Md-250



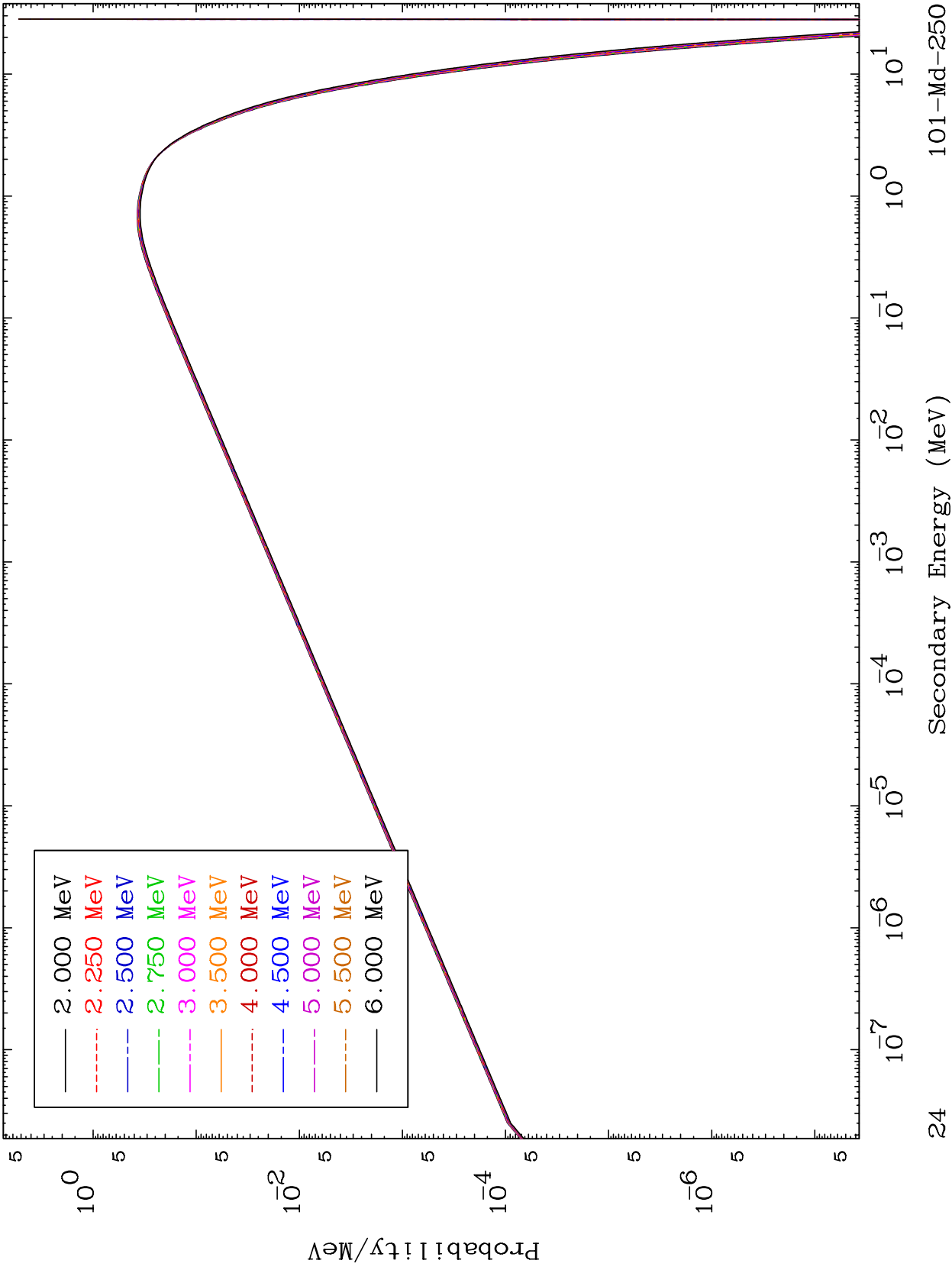
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Fission
Energy Distribution

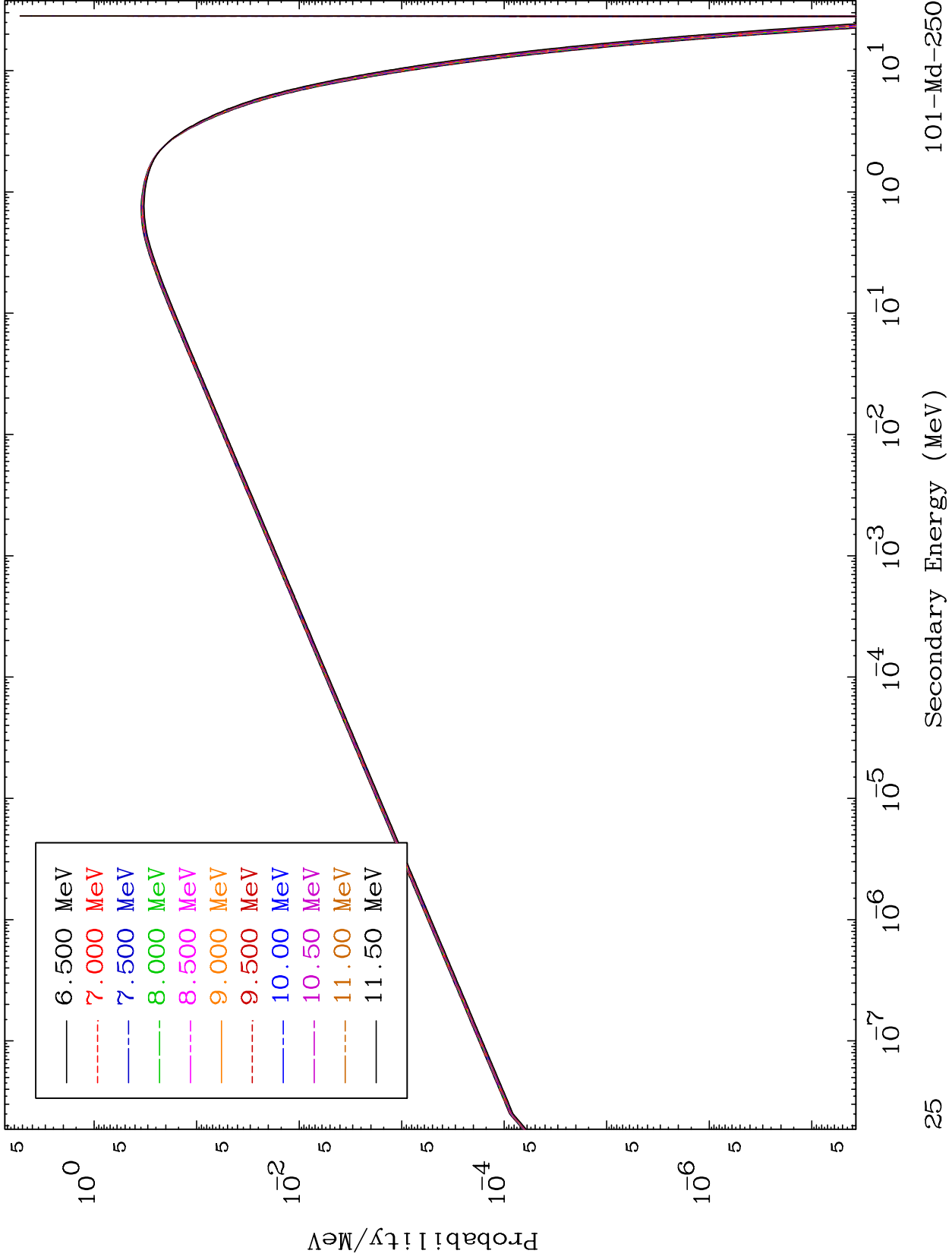
101-Md-250



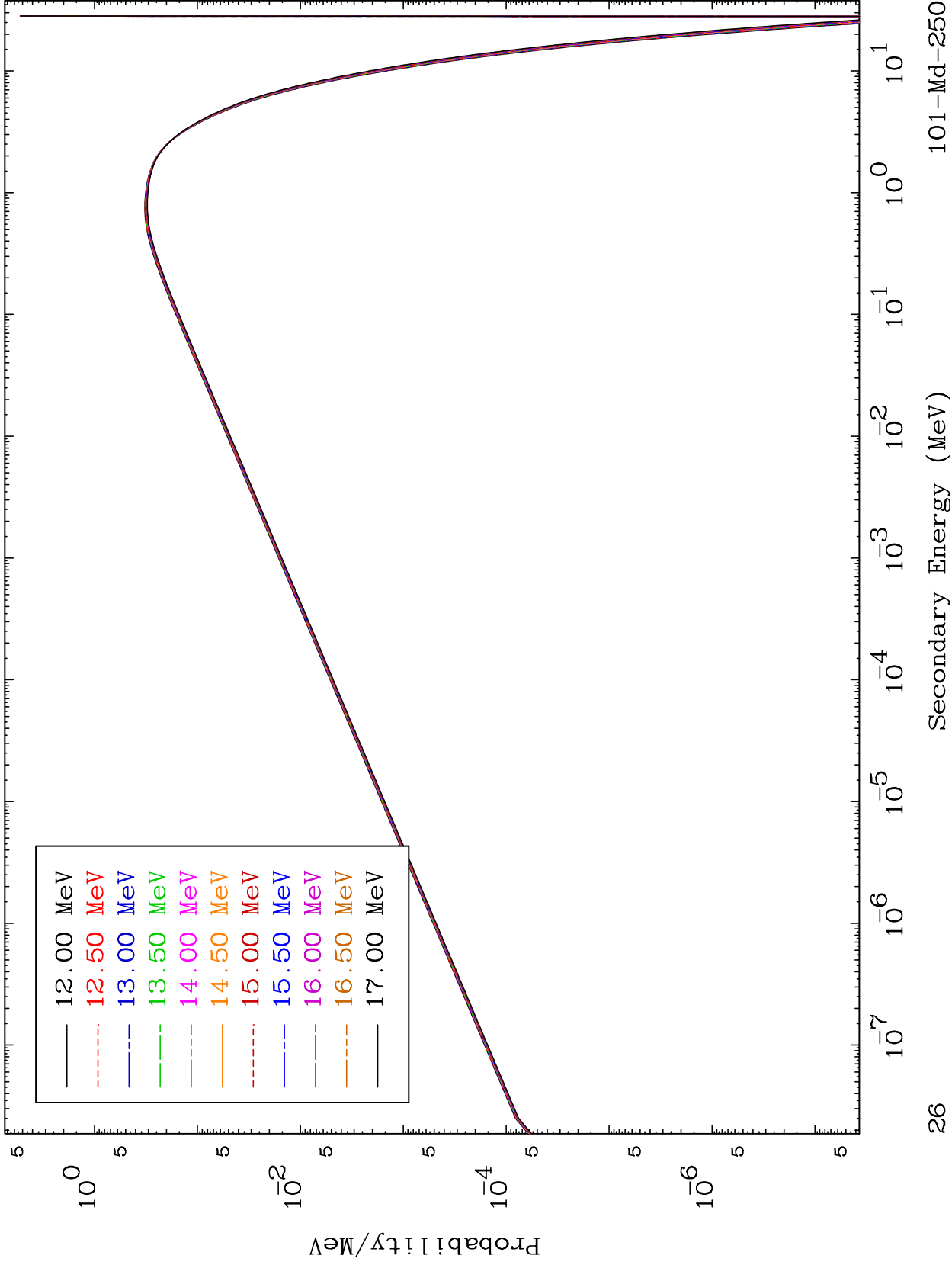
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Fission Energy Distribution

101-Md-250



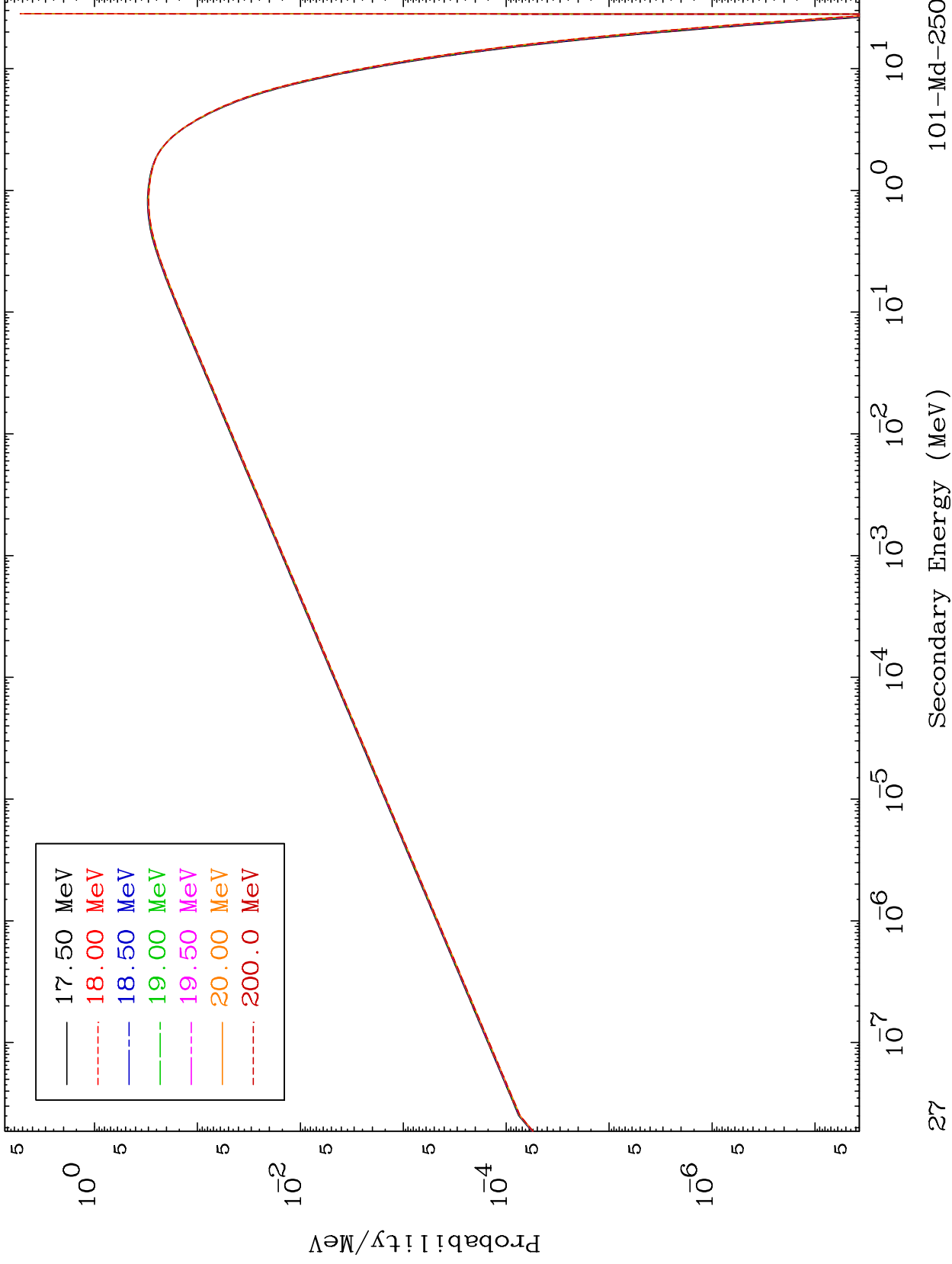
Fission
Energy Distribution



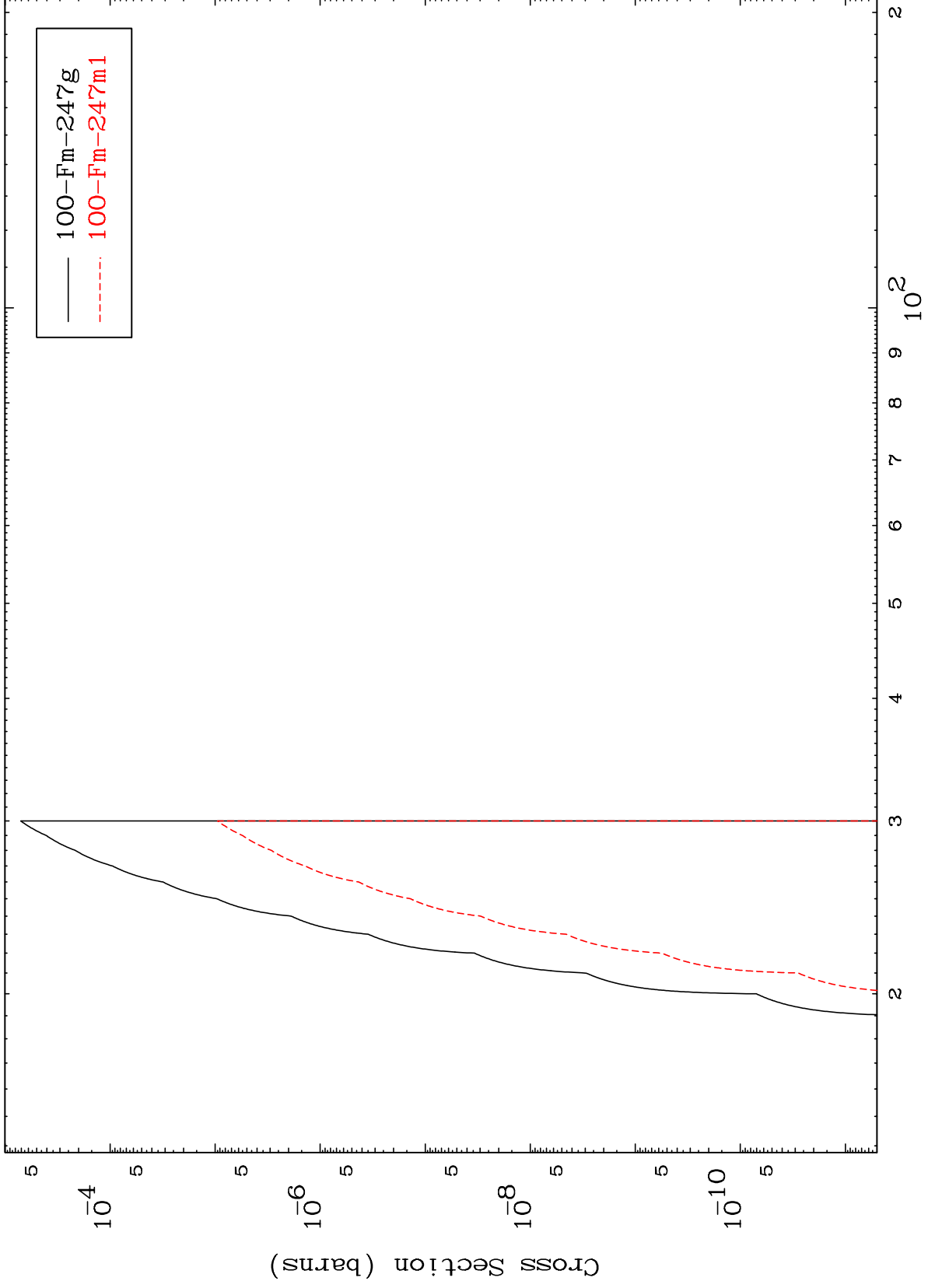
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Fission
Energy Distribution

101-Md-250



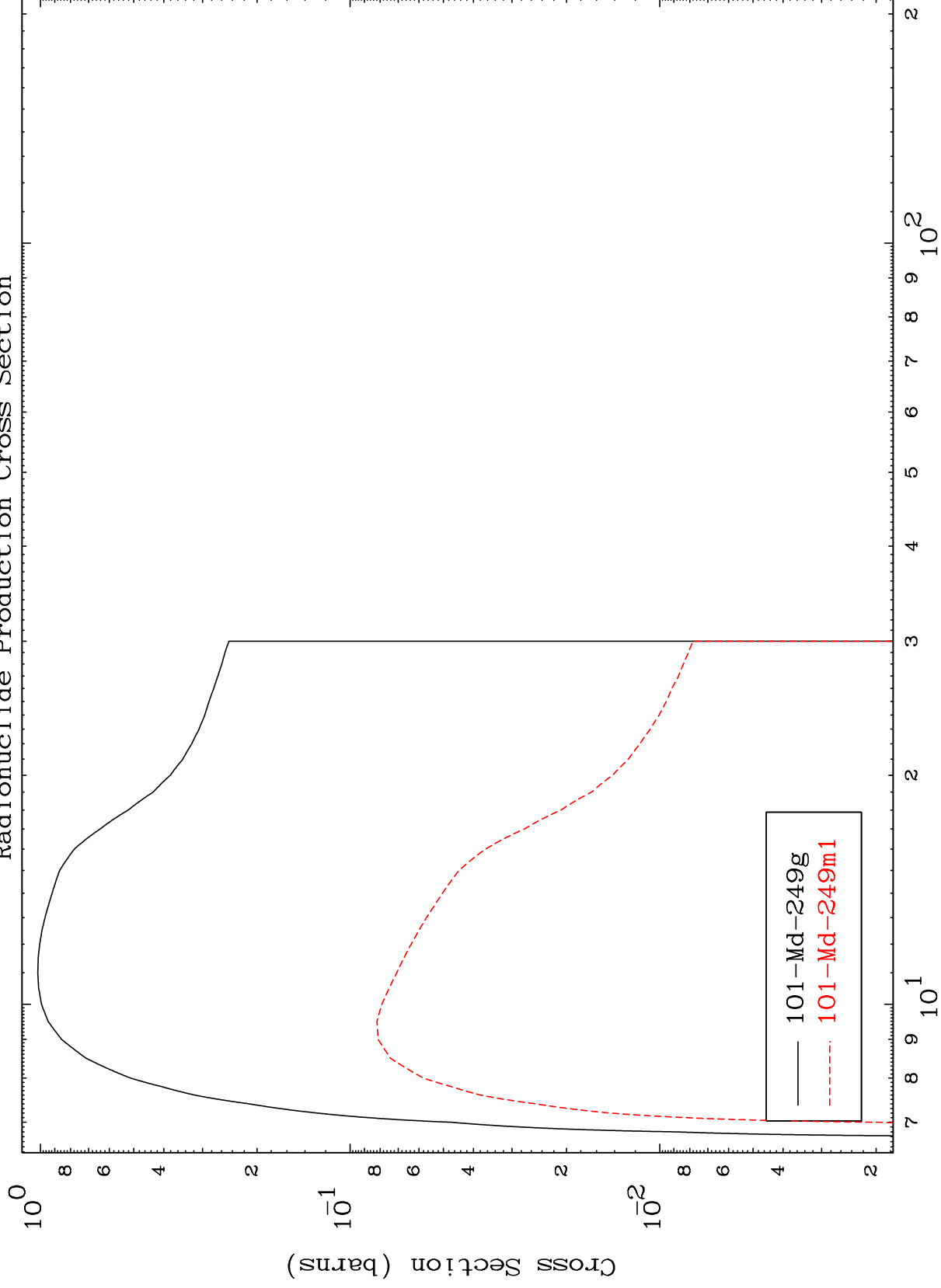
Radionuclide Production Cross Section



MAT 9951

101-Md-250

(n,2n)
Radionuclide Production Cross Section



101-Md-250

Incident Energy (MeV)

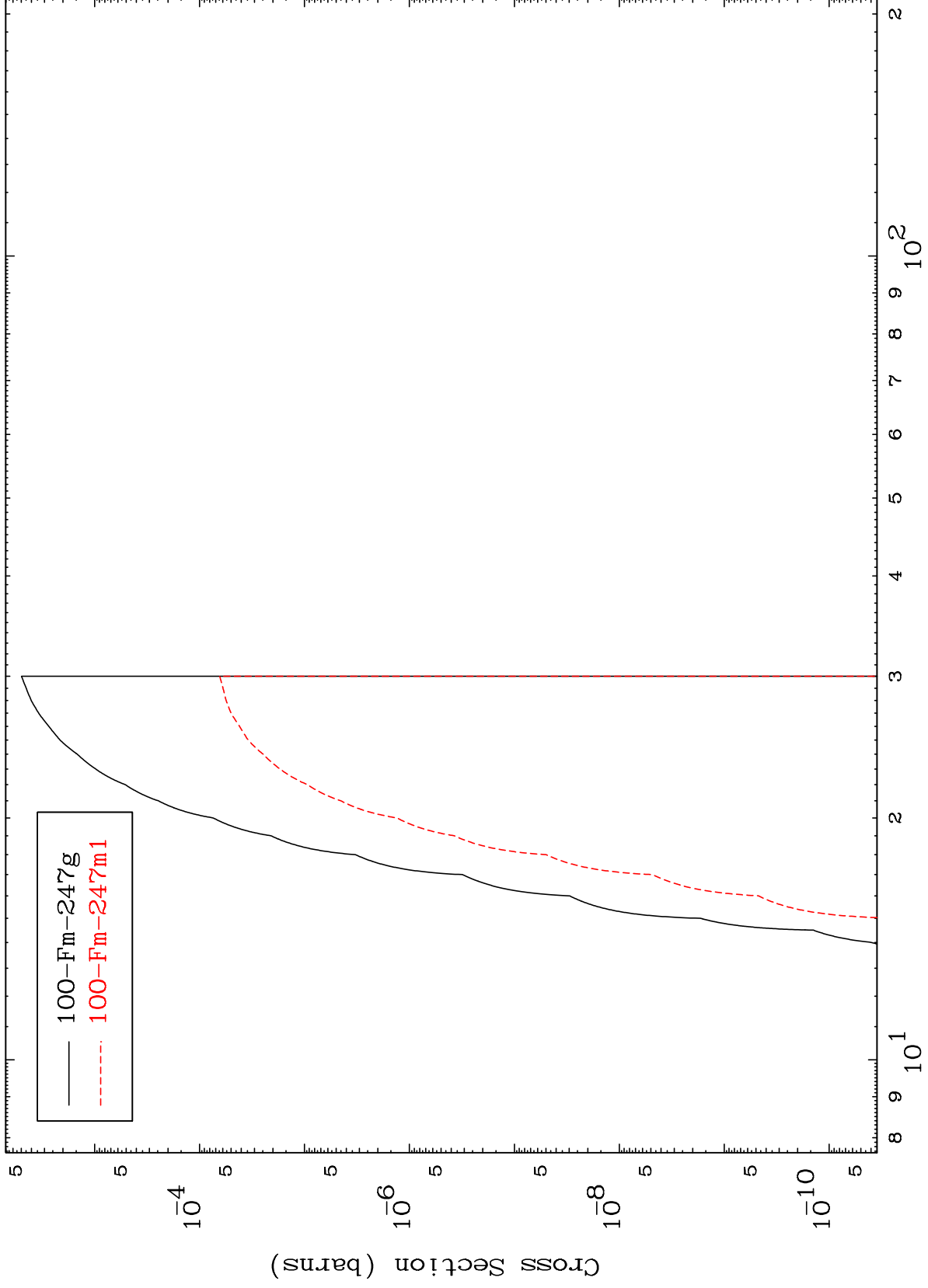
29

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(n,n') t

101-Md-250

Radionuclide Production Cross Section



100-Fm-247g
100-Fm-247m1

Incident Energy (MeV)

101-Md-250

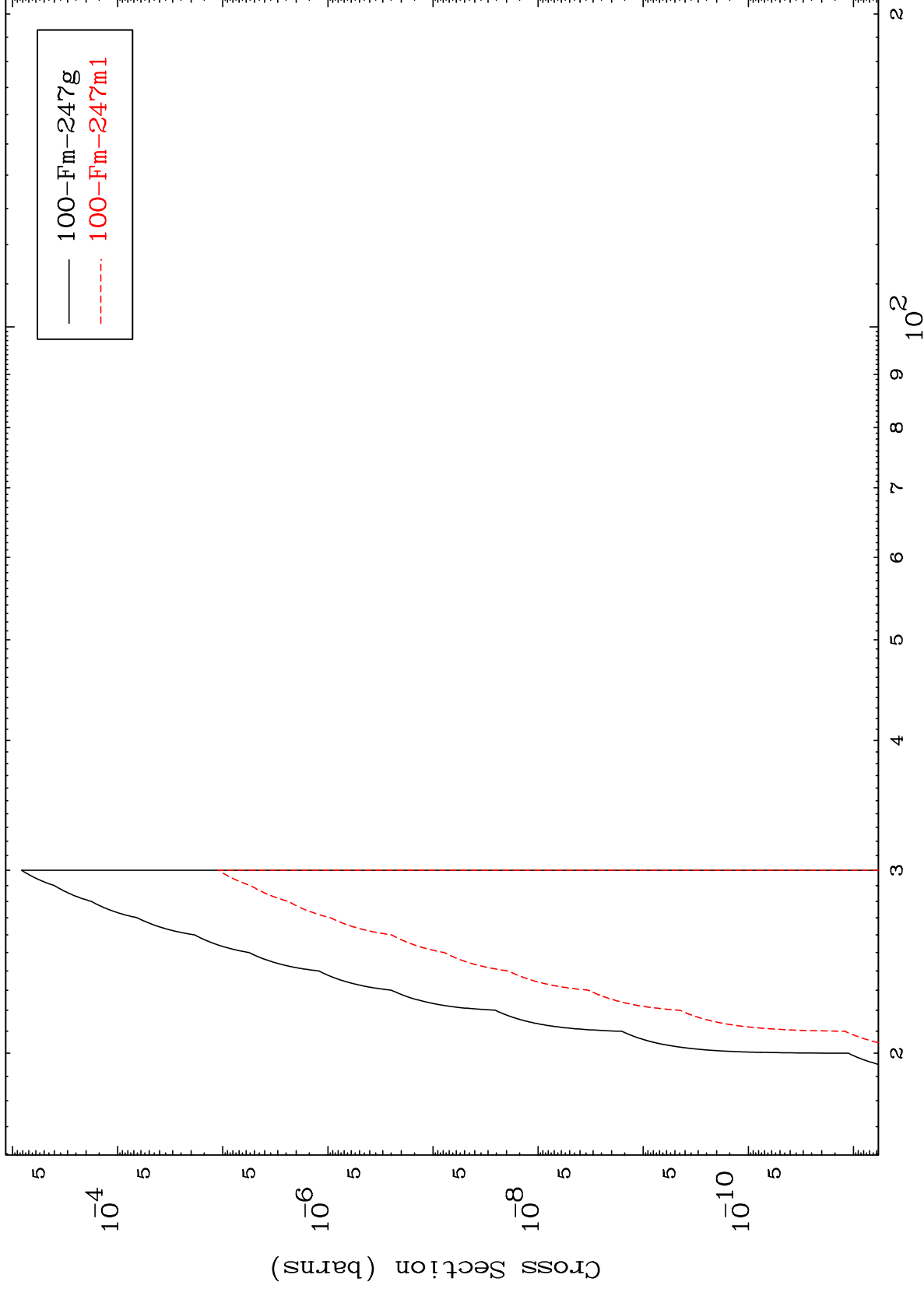
30

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(n,3n) p

101-Md-250

Radionuclide Production Cross Section



31

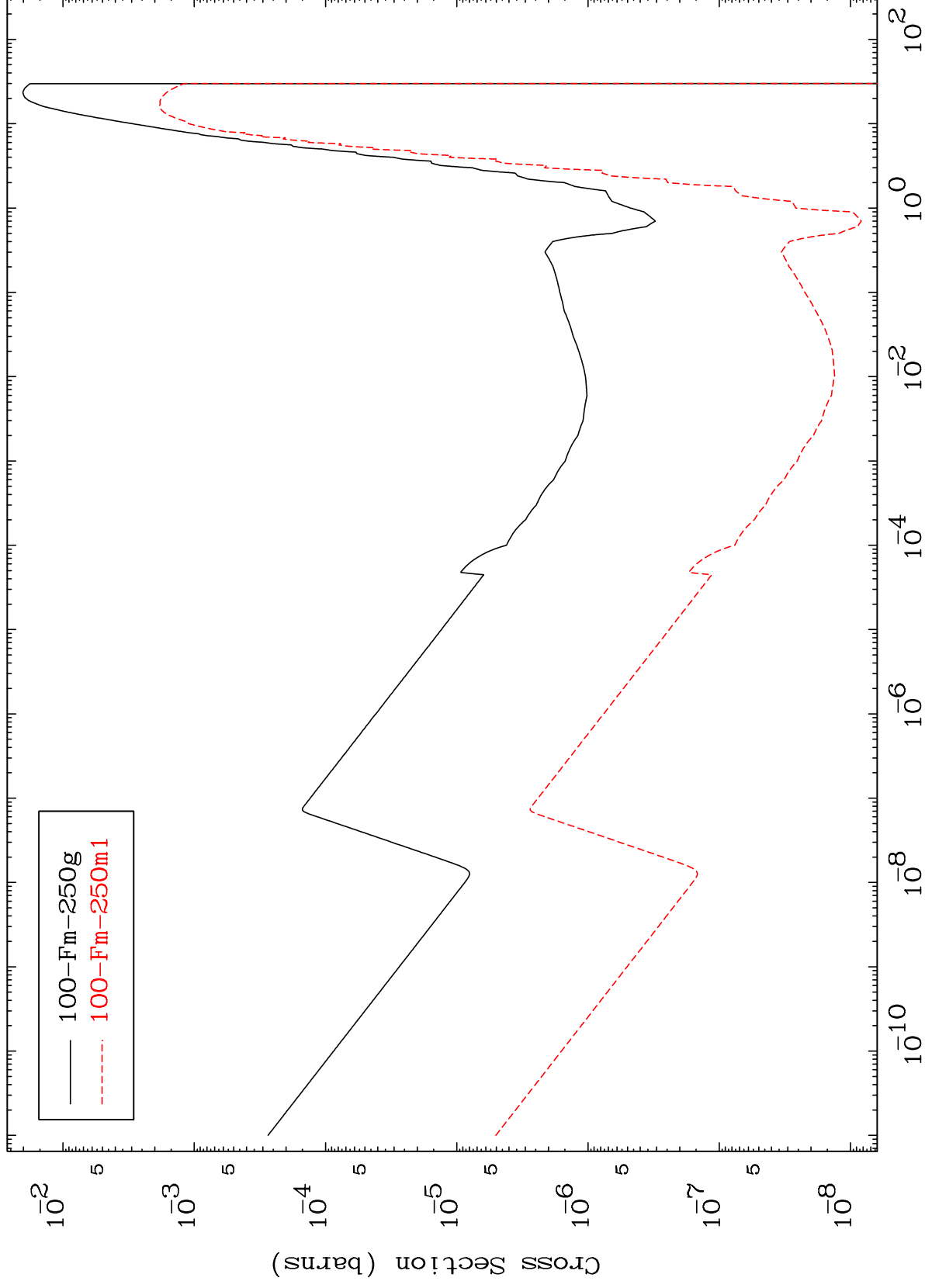
Incident Energy (MeV)

101-Md-250

MAT 9951

101-Md-250

(n,p)
Radionuclide Production Cross Section



32

101-Md-250

Incident Energy (MeV)