

Program EVALPLOT  
(Version 2015-2)

by

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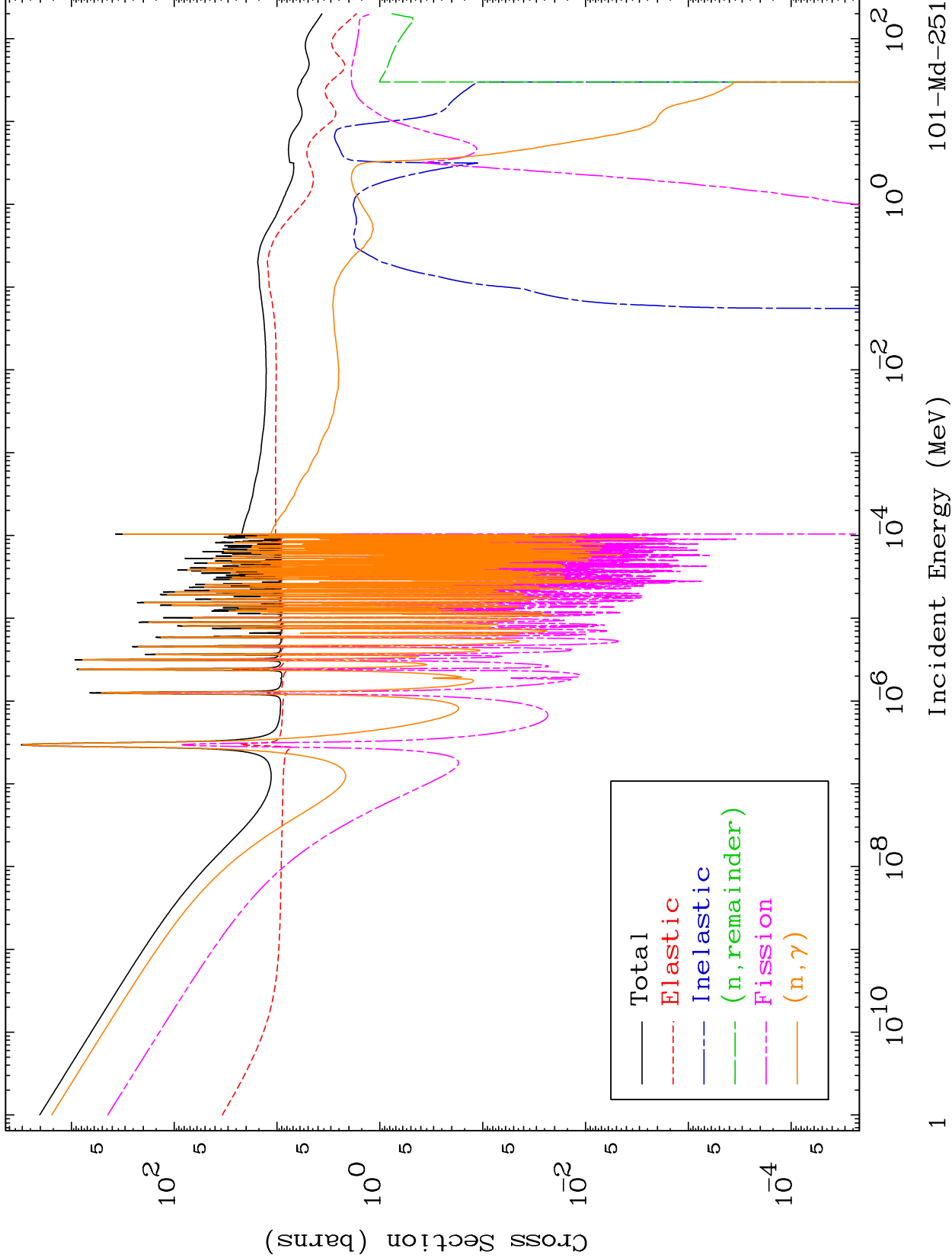
Press Mouse Button to Start

MAT 9952

Major

101-Md-251

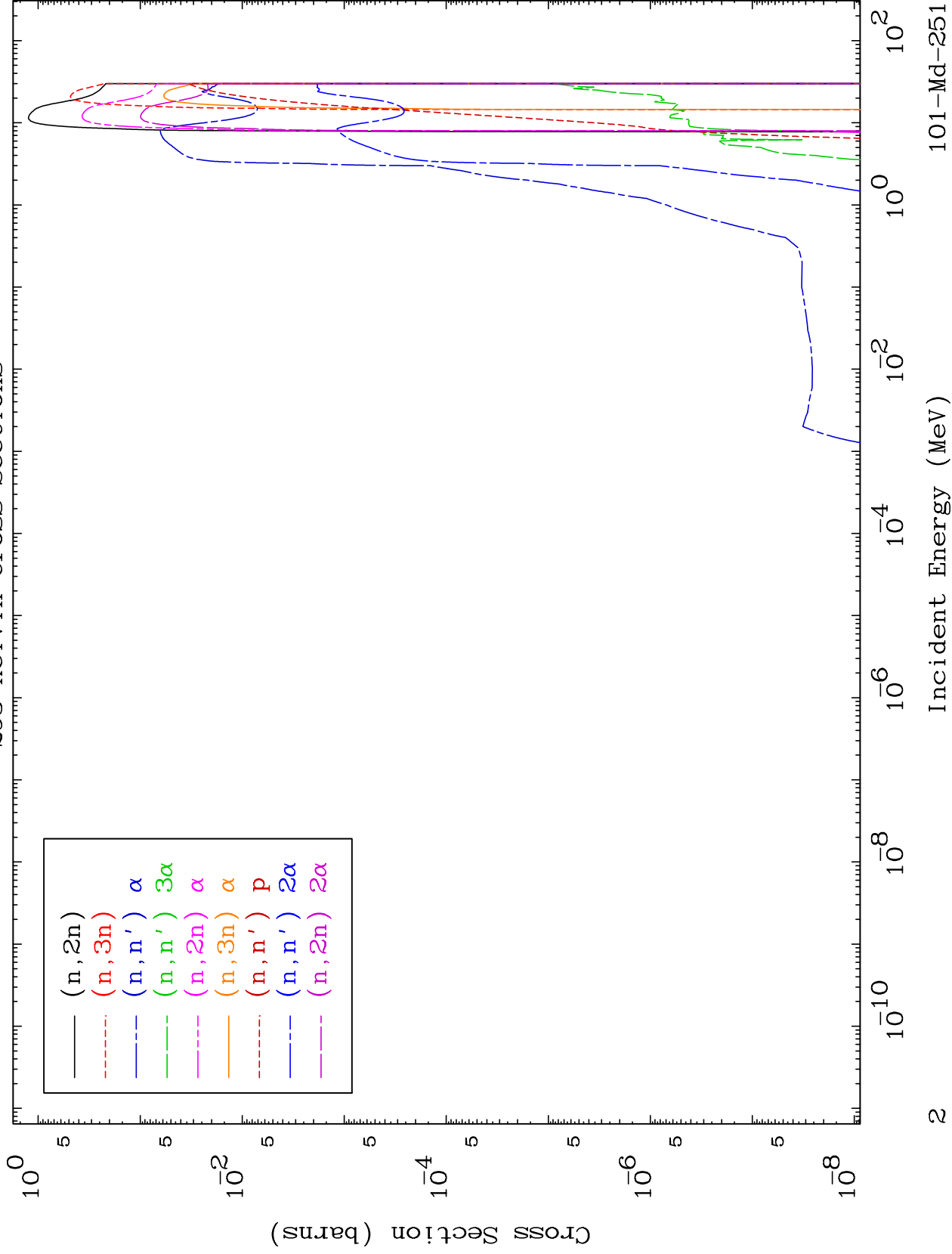
293 Kelvin Cross Sections



MAT 9952

Neutron Production  
293 Kelvin Cross Sections

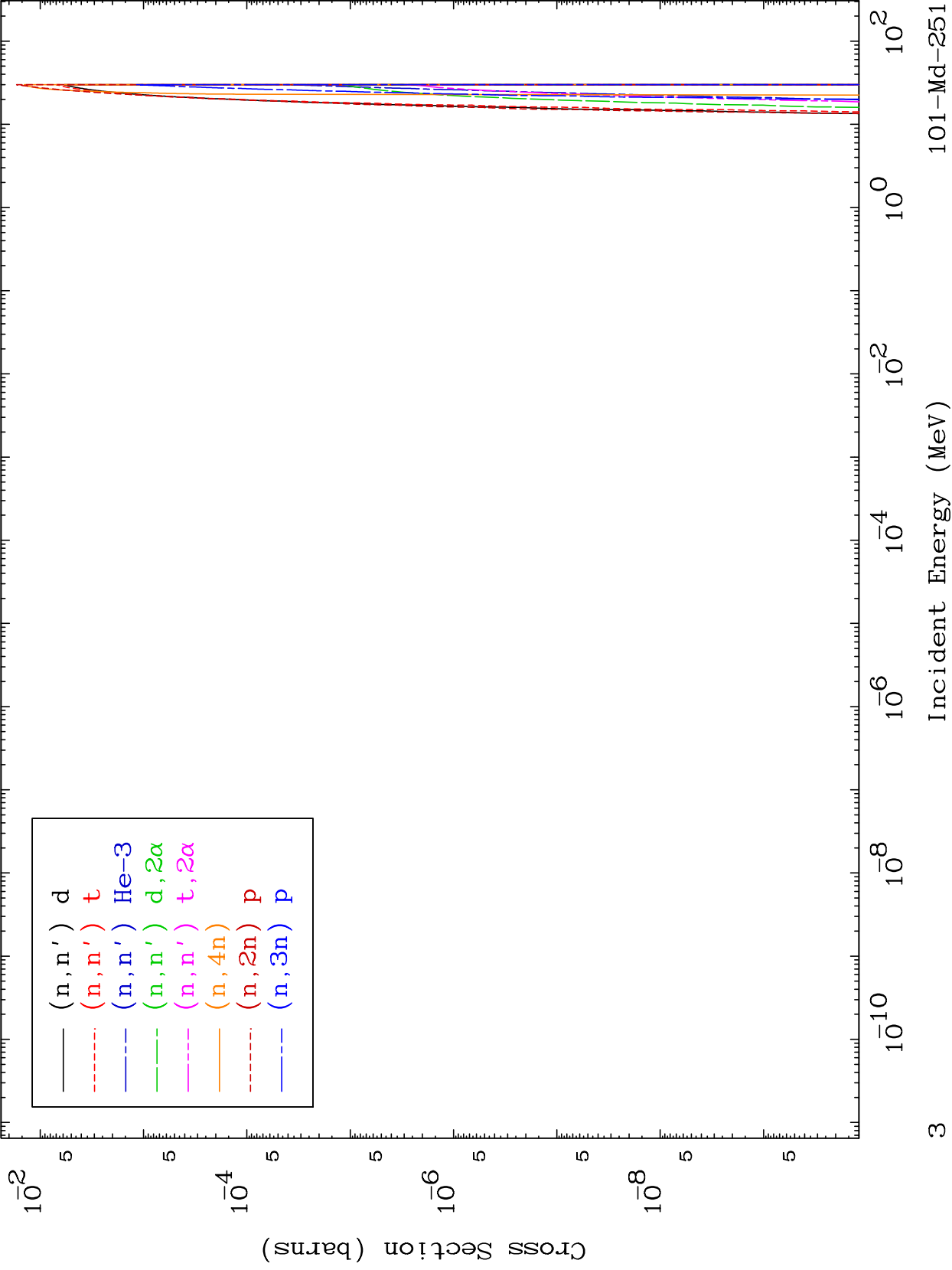
101-Md-251



MAT 9952

Neutron Production  
293 Kelvin Cross Sections

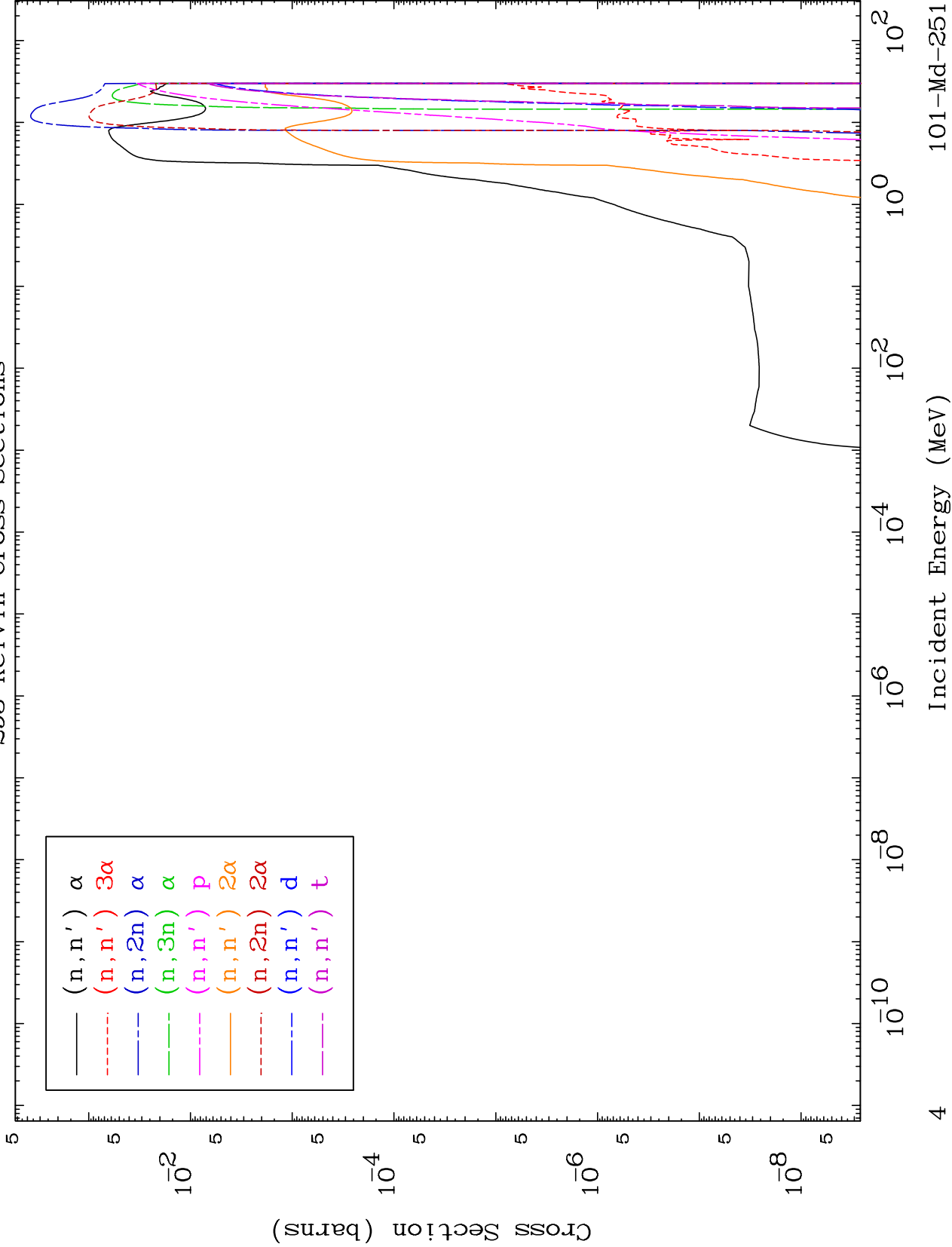
101-Md-251



MAT 9952

Charged Particle  
293 Kelvin Cross Sections

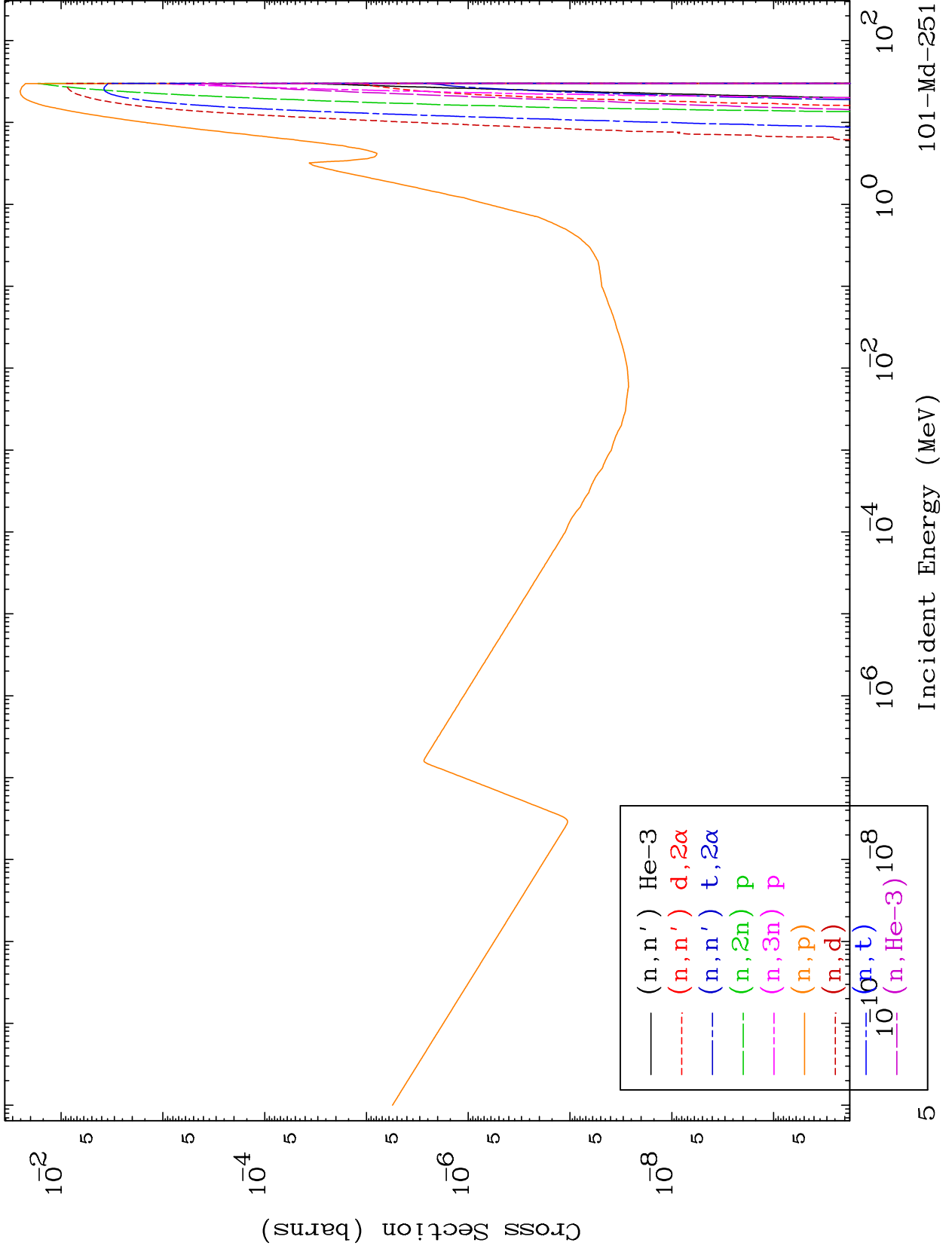
101-Md-251

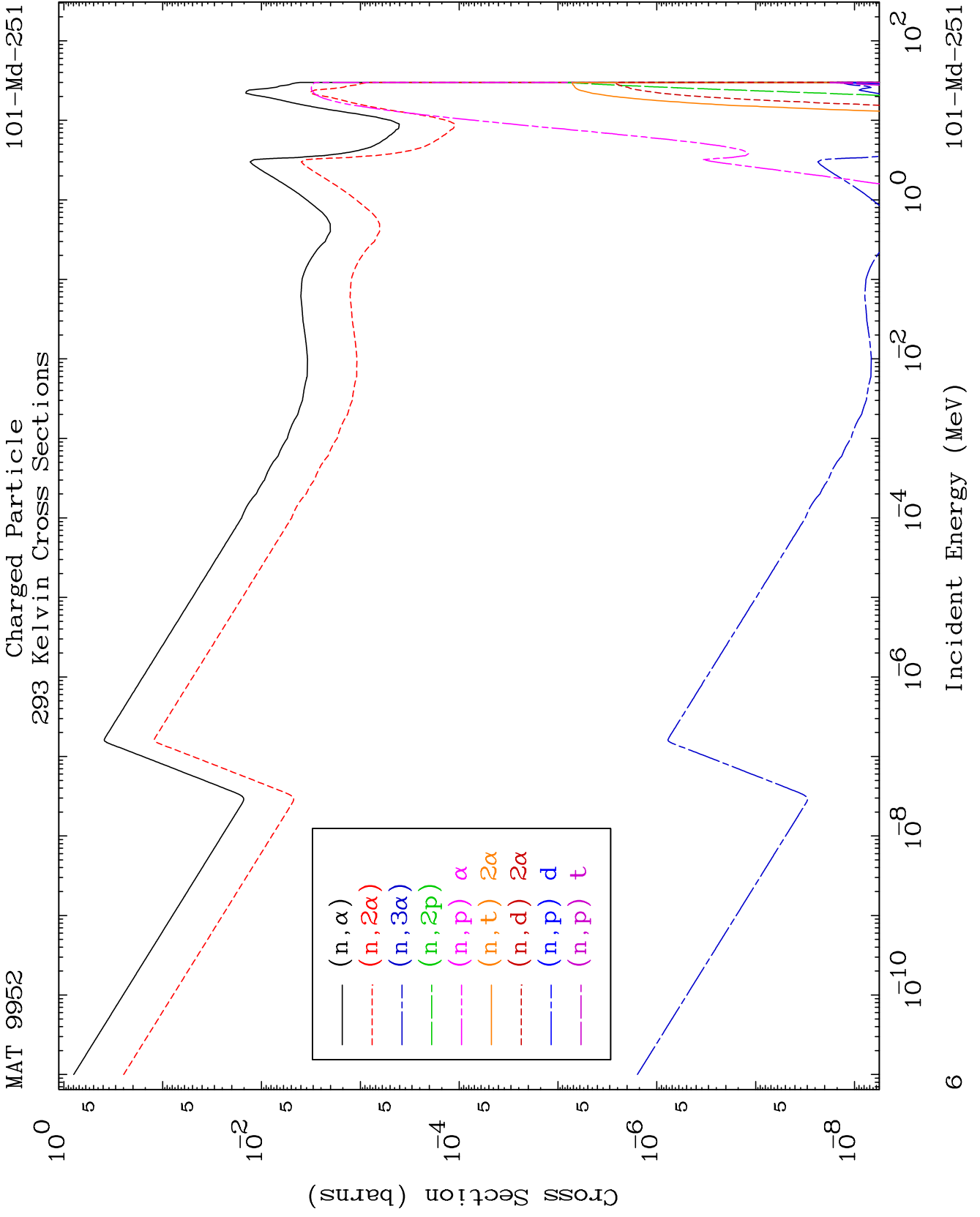


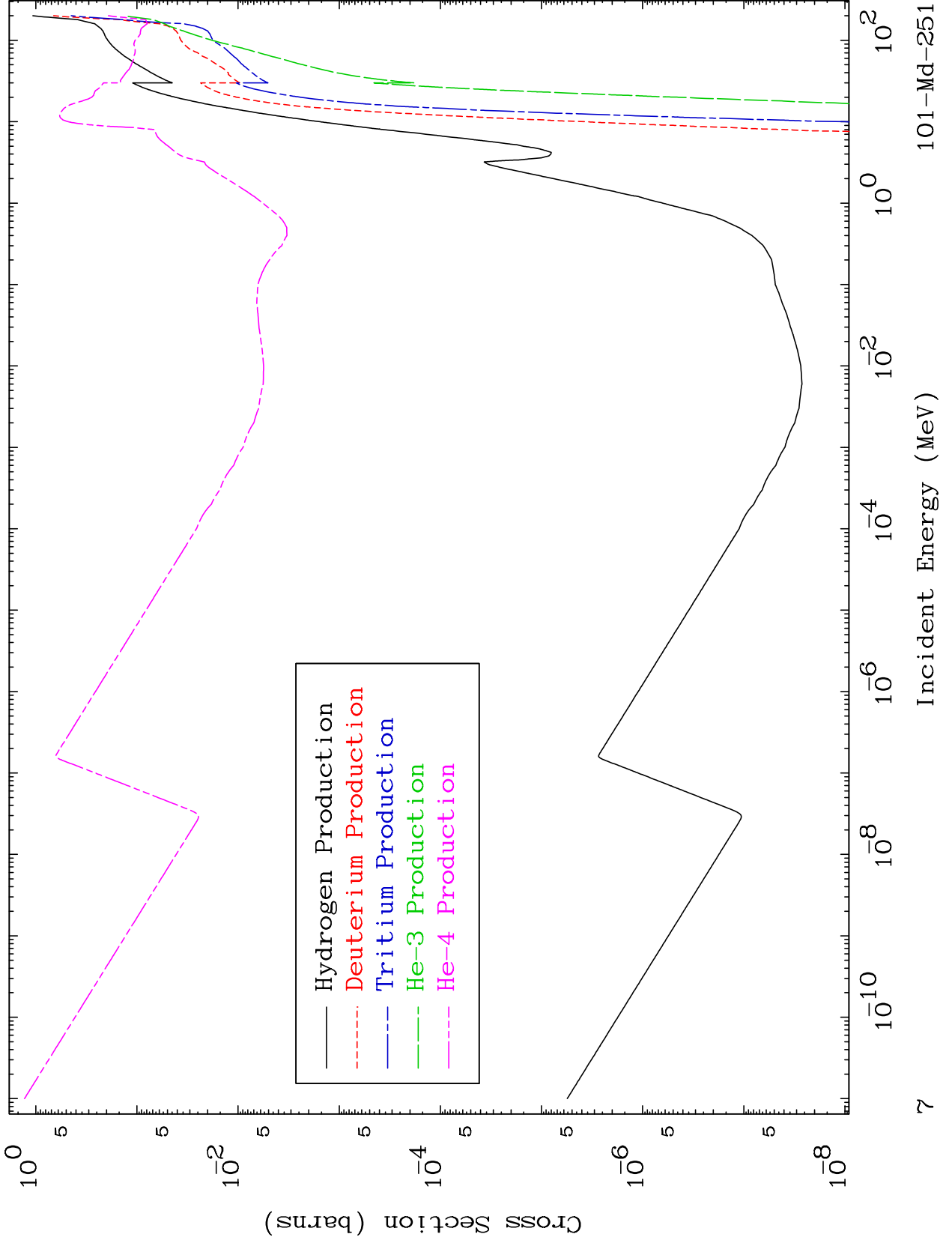
MAT 9952

Charged Particle  
293 Kelvin Cross Sections

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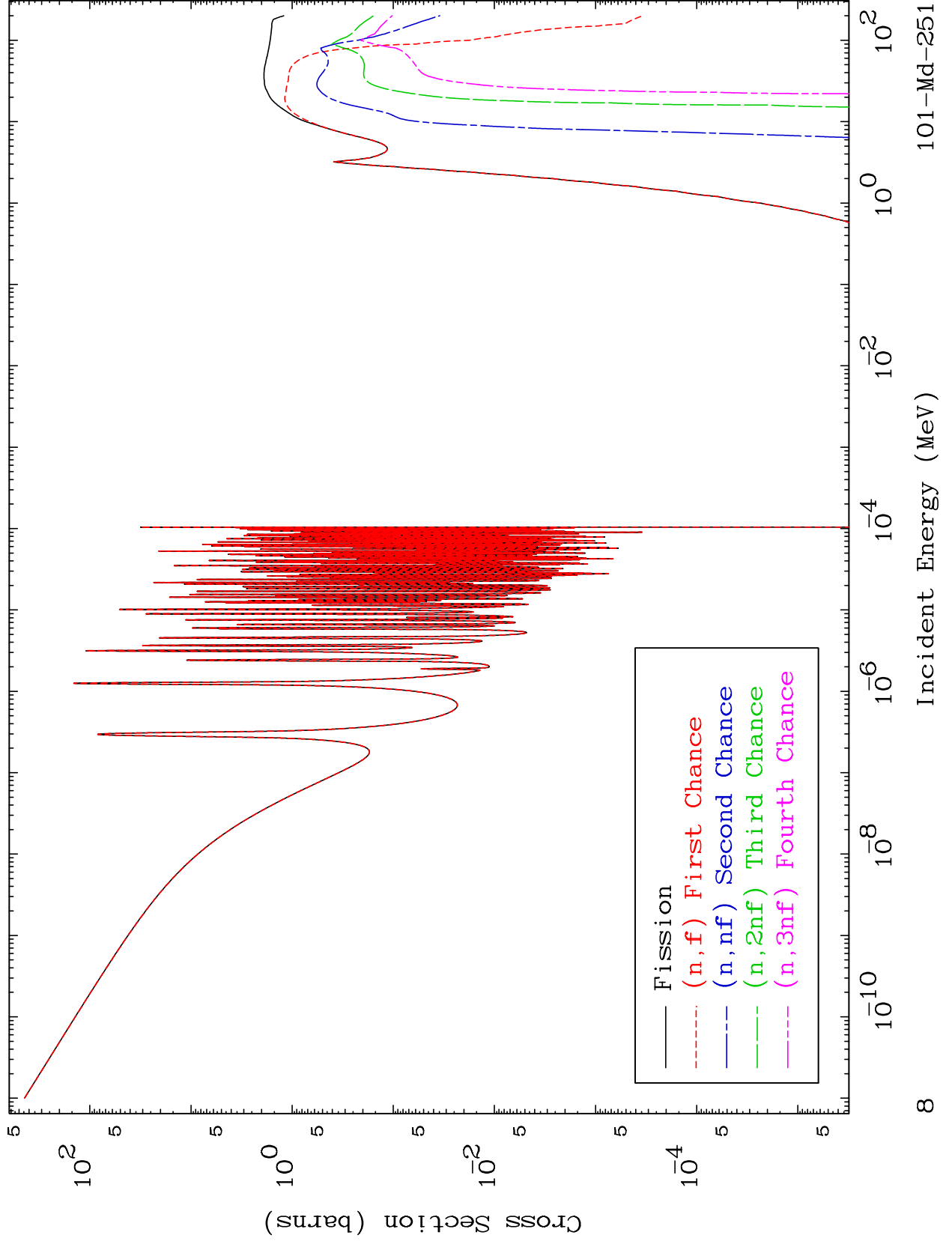




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Fission  
293 Kelvin Cross Sections

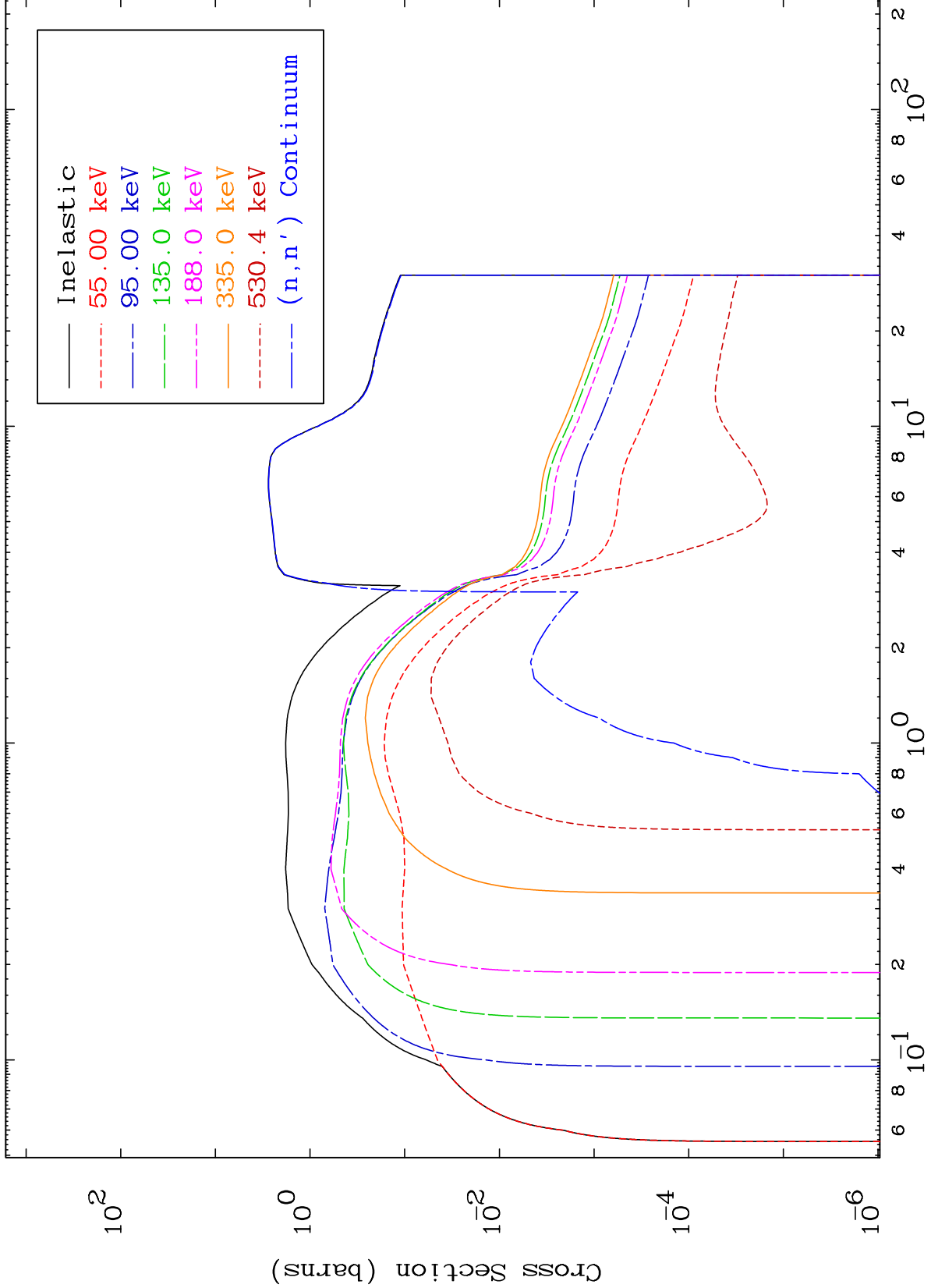
101-Md-251



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(n,n') Level  
293 Kelvin Cross Sections

101-Md-251



9

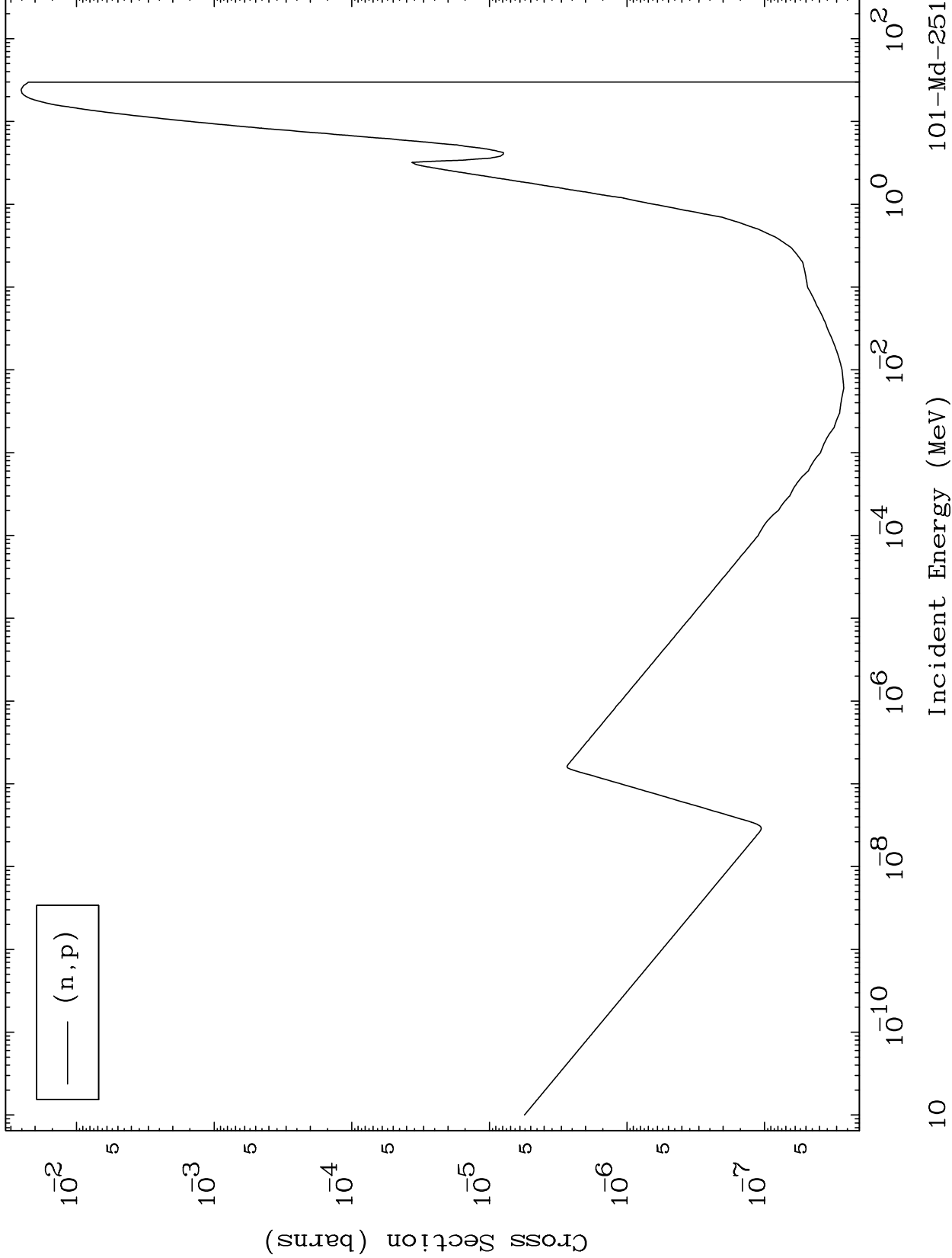
Incident Energy (MeV)

101-Md-251

MAT 9952

(n,p) Levels  
293 Kelvin Cross Sections

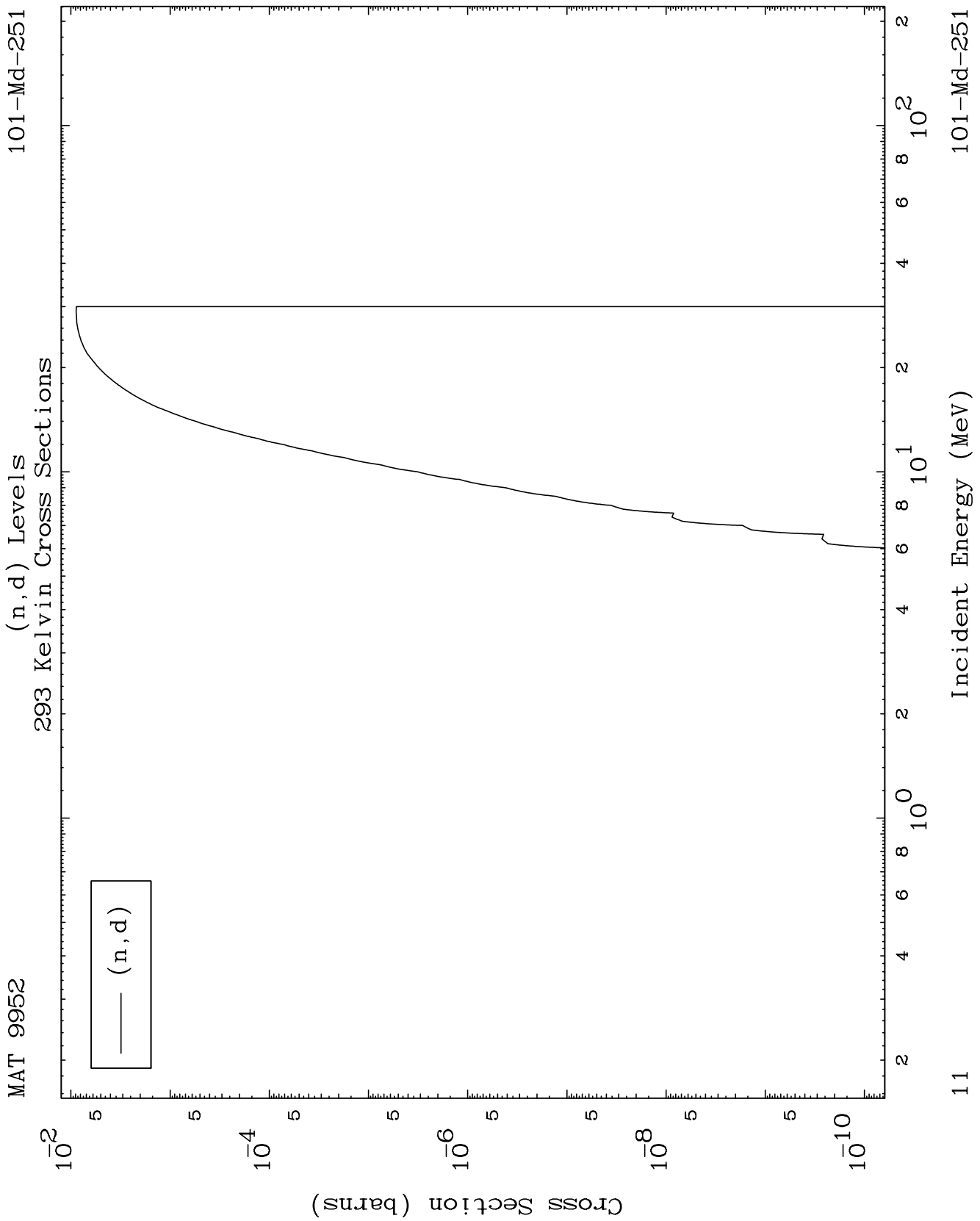
101-Md-251



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Incident Energy (MeV)

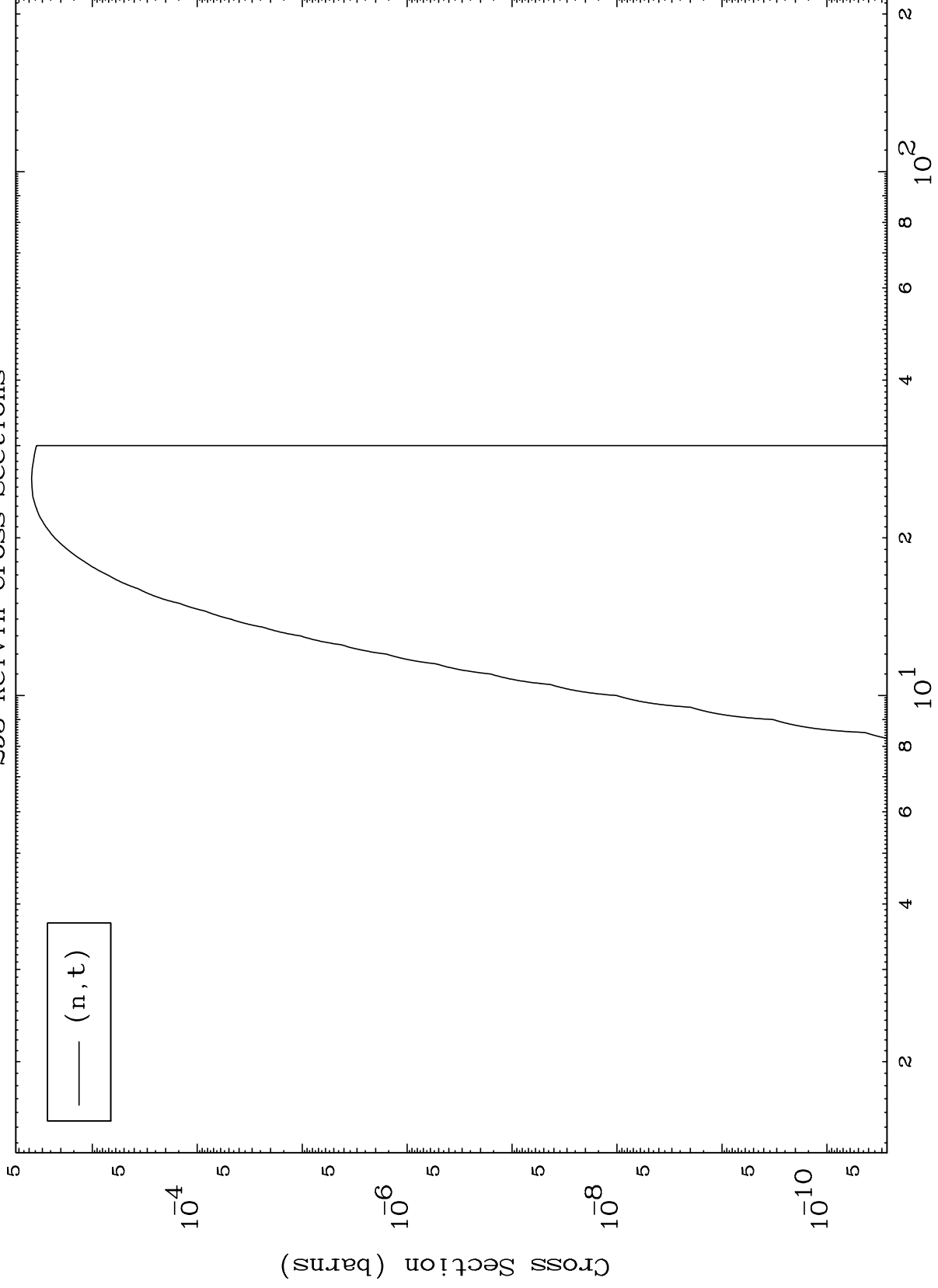
101-Md-251



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(n,t) Levels  
293 Kelvin Cross Sections

101-Md-251



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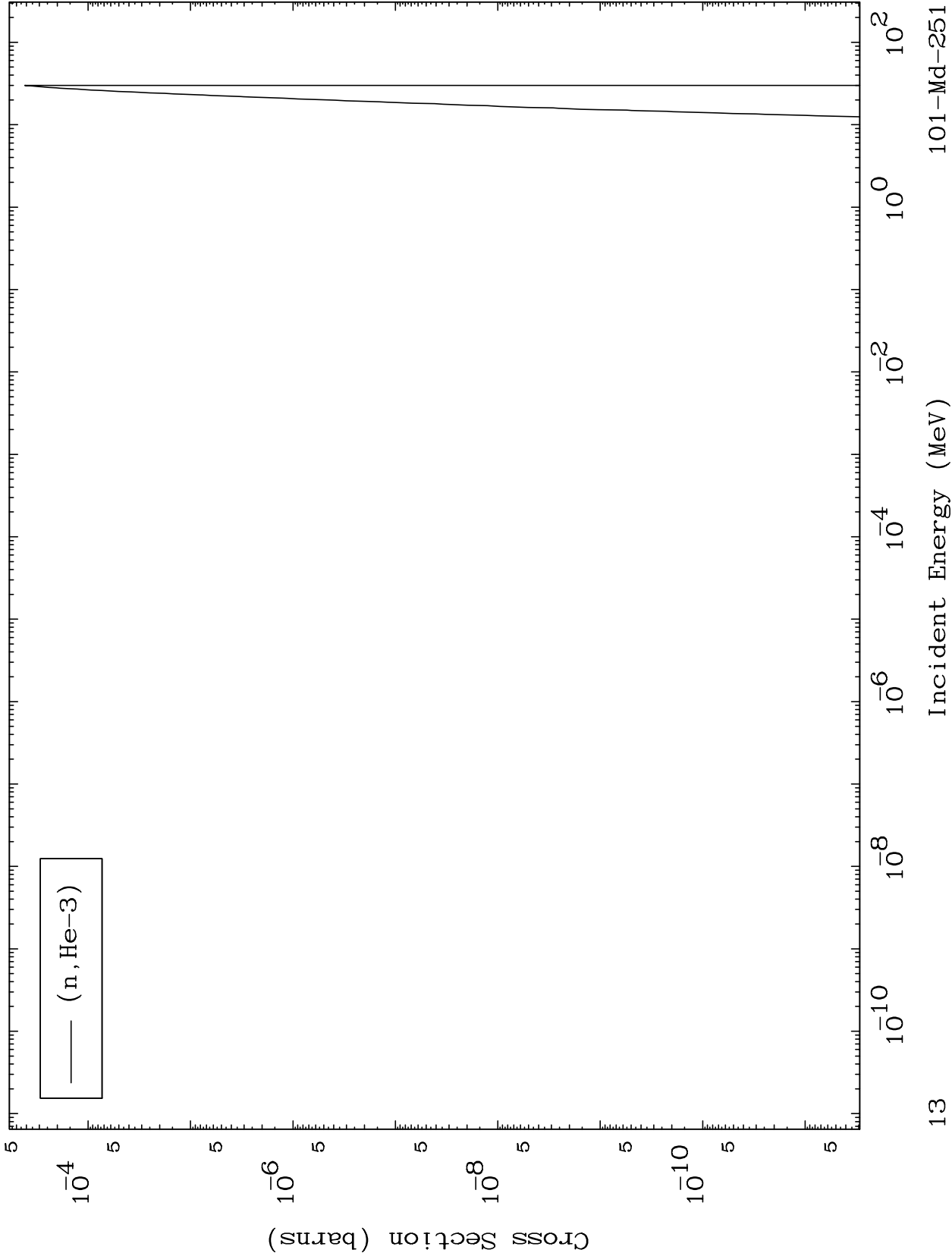
Incident Energy (MeV)

101-Md-251

MAT 9952

(n,He3) Levels  
293 Kelvin Cross Sections

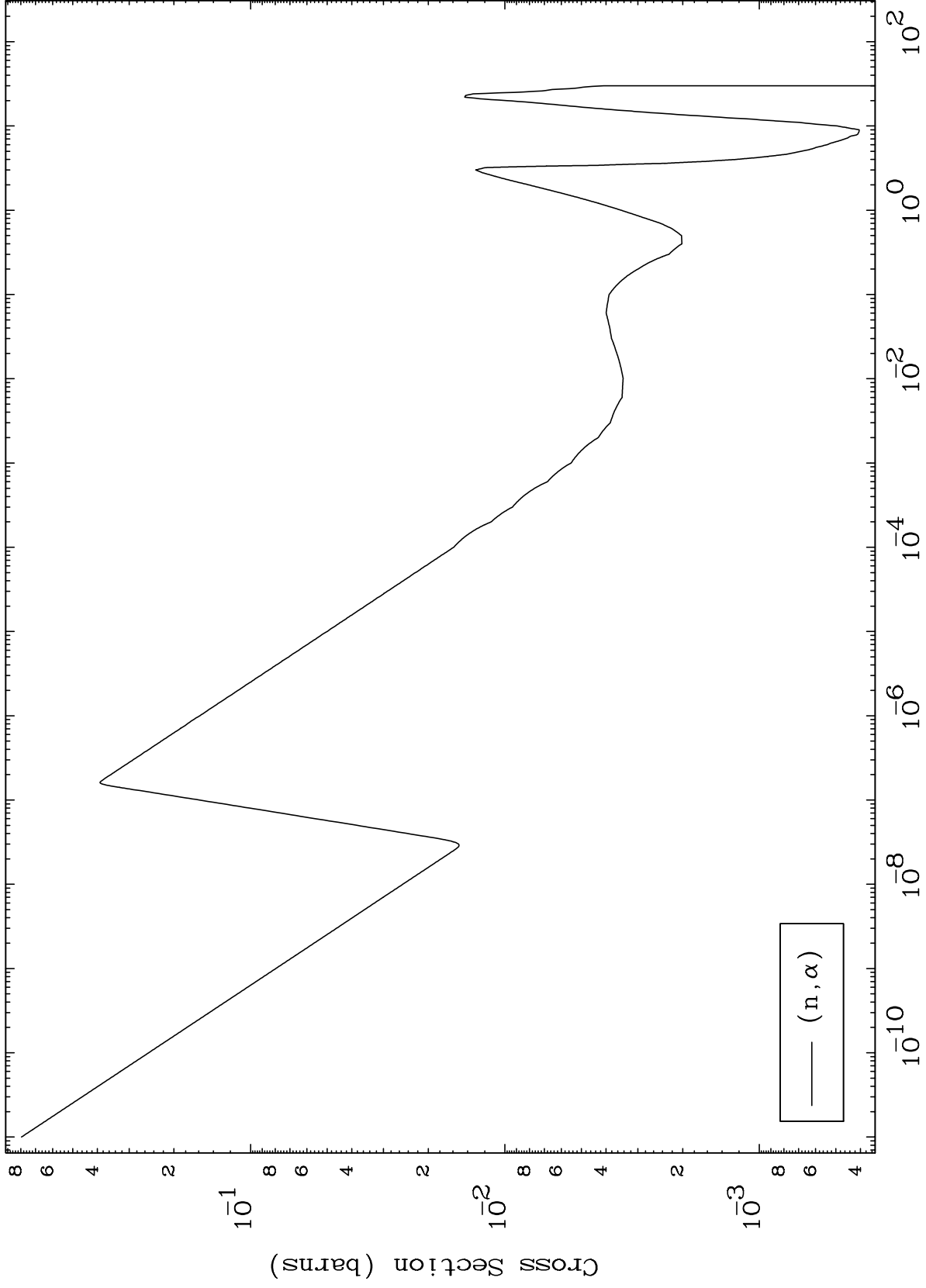
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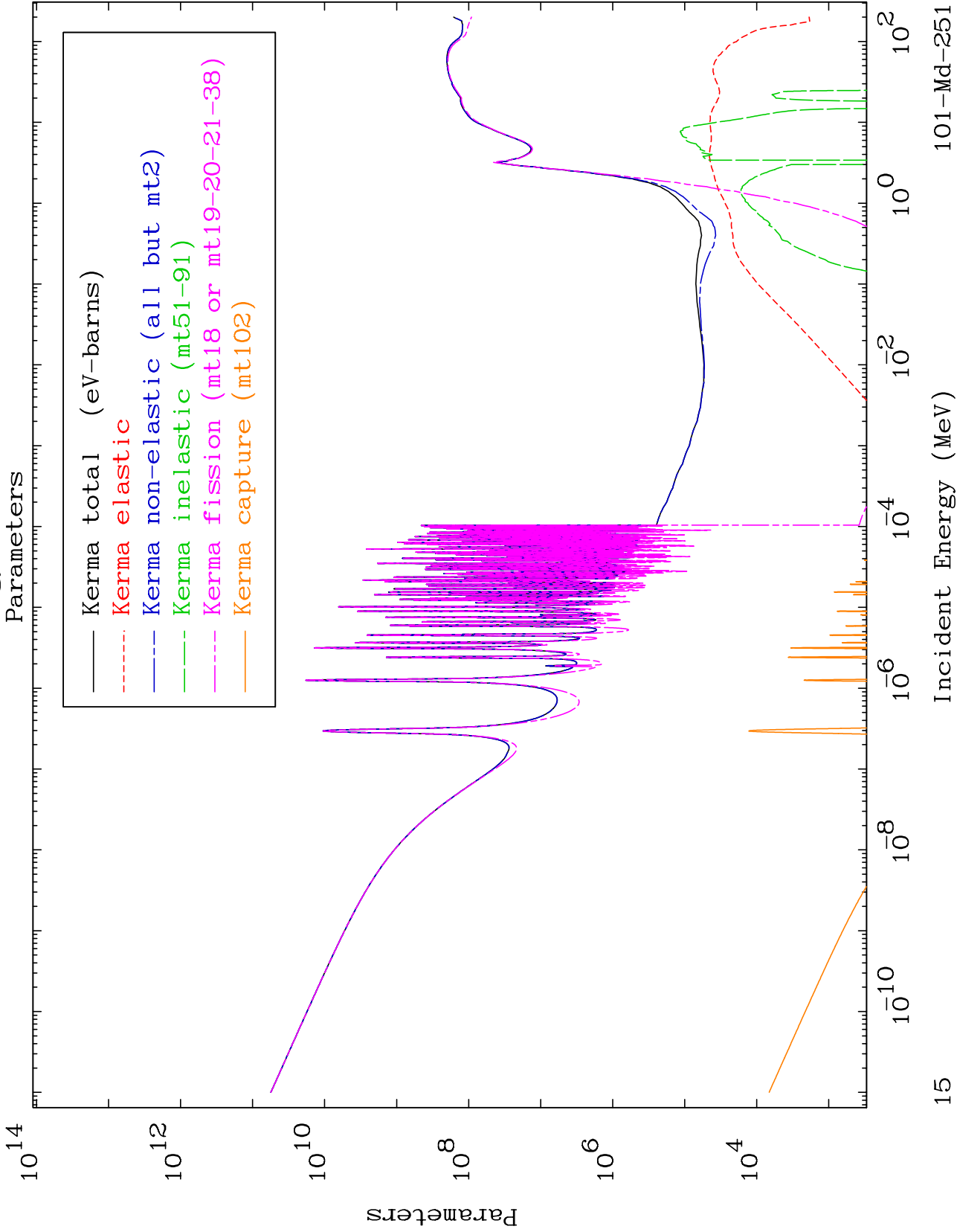


MAT 9952

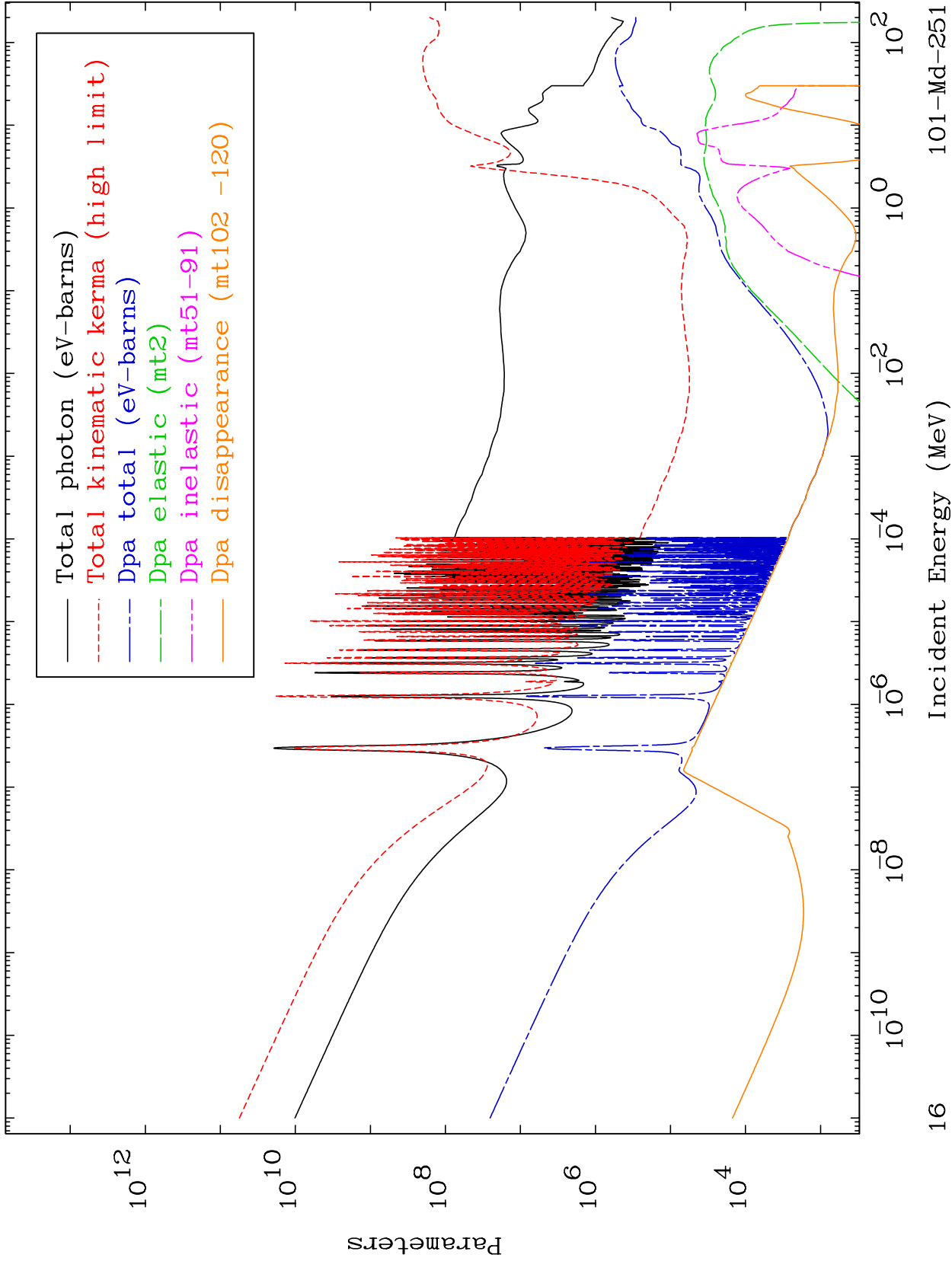
(n,α) Levels  
293 Kelvin Cross Sections

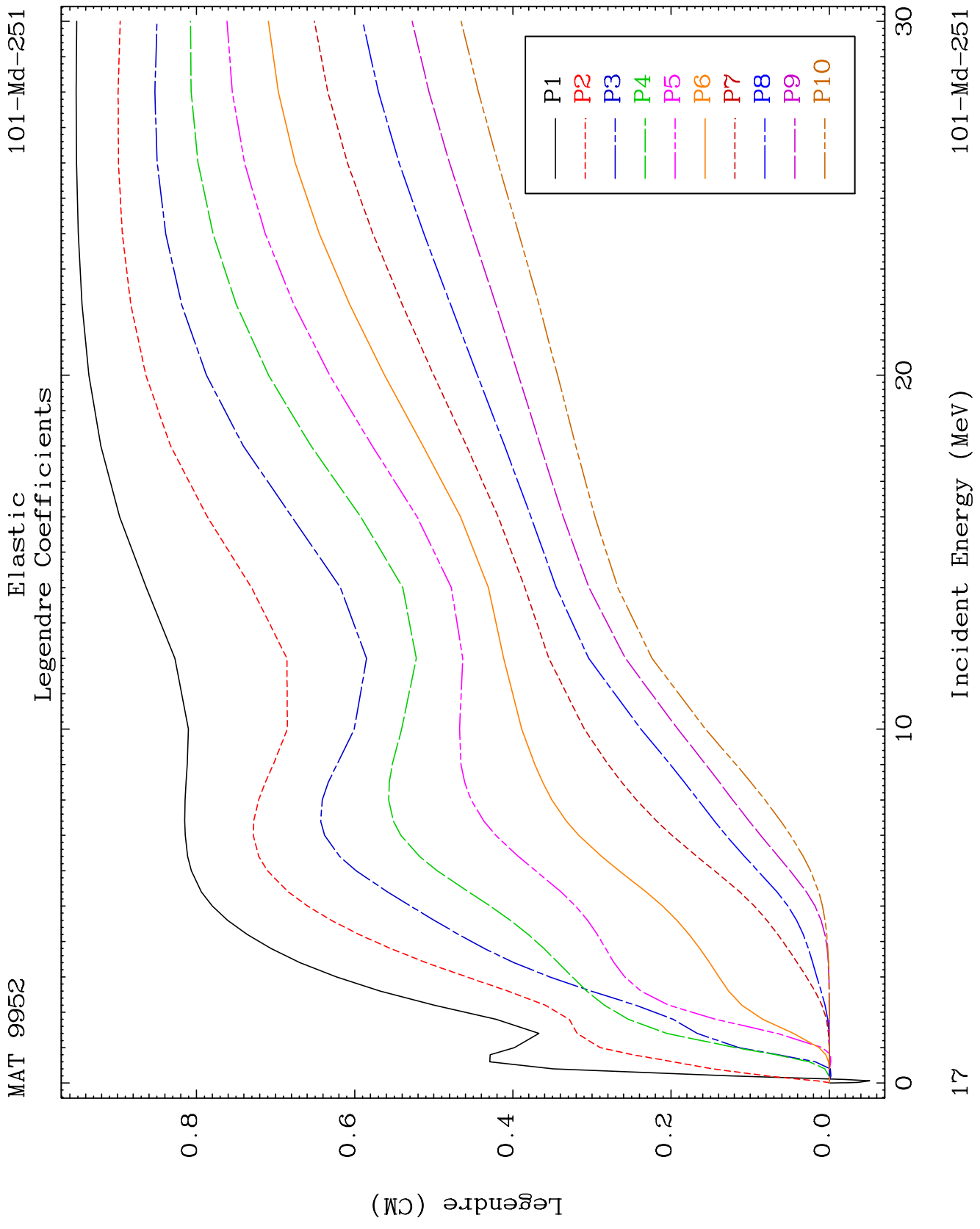
101-Md-251







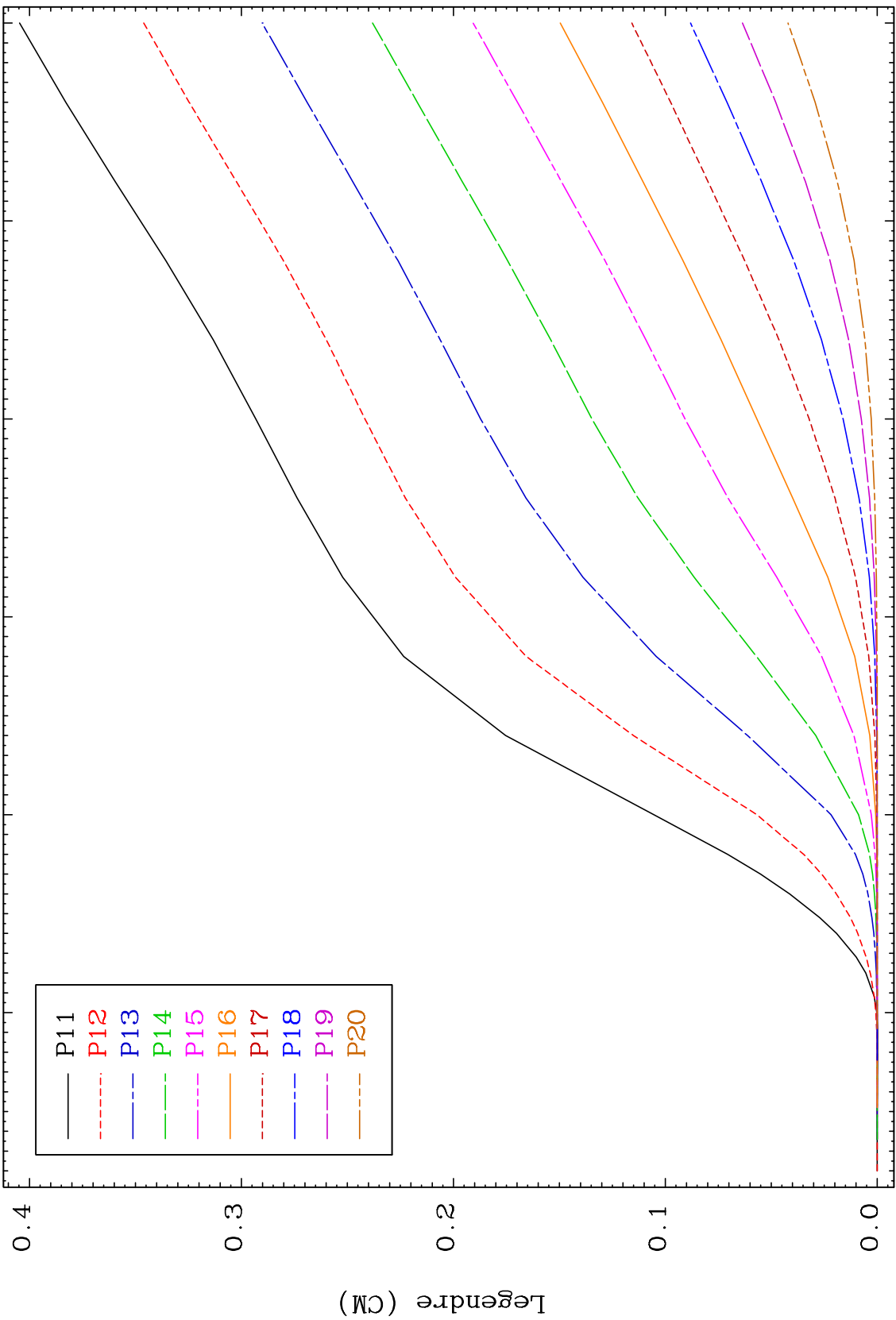




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Elastic Legendre Coefficients

101-Md-251



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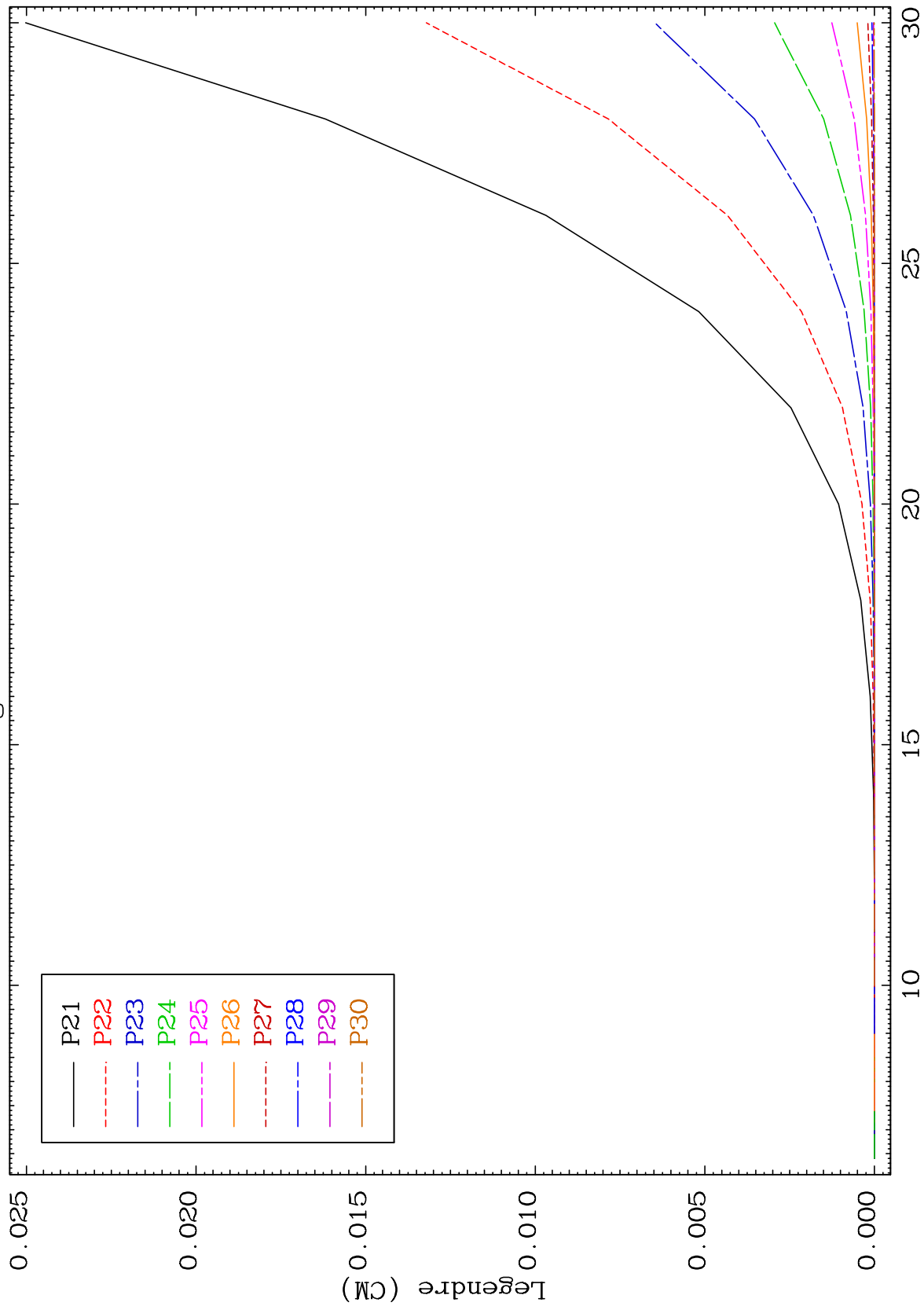
Incident Energy (MeV)

101-Md-251

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Elastic Legendre Coefficients

101-Md-251



19

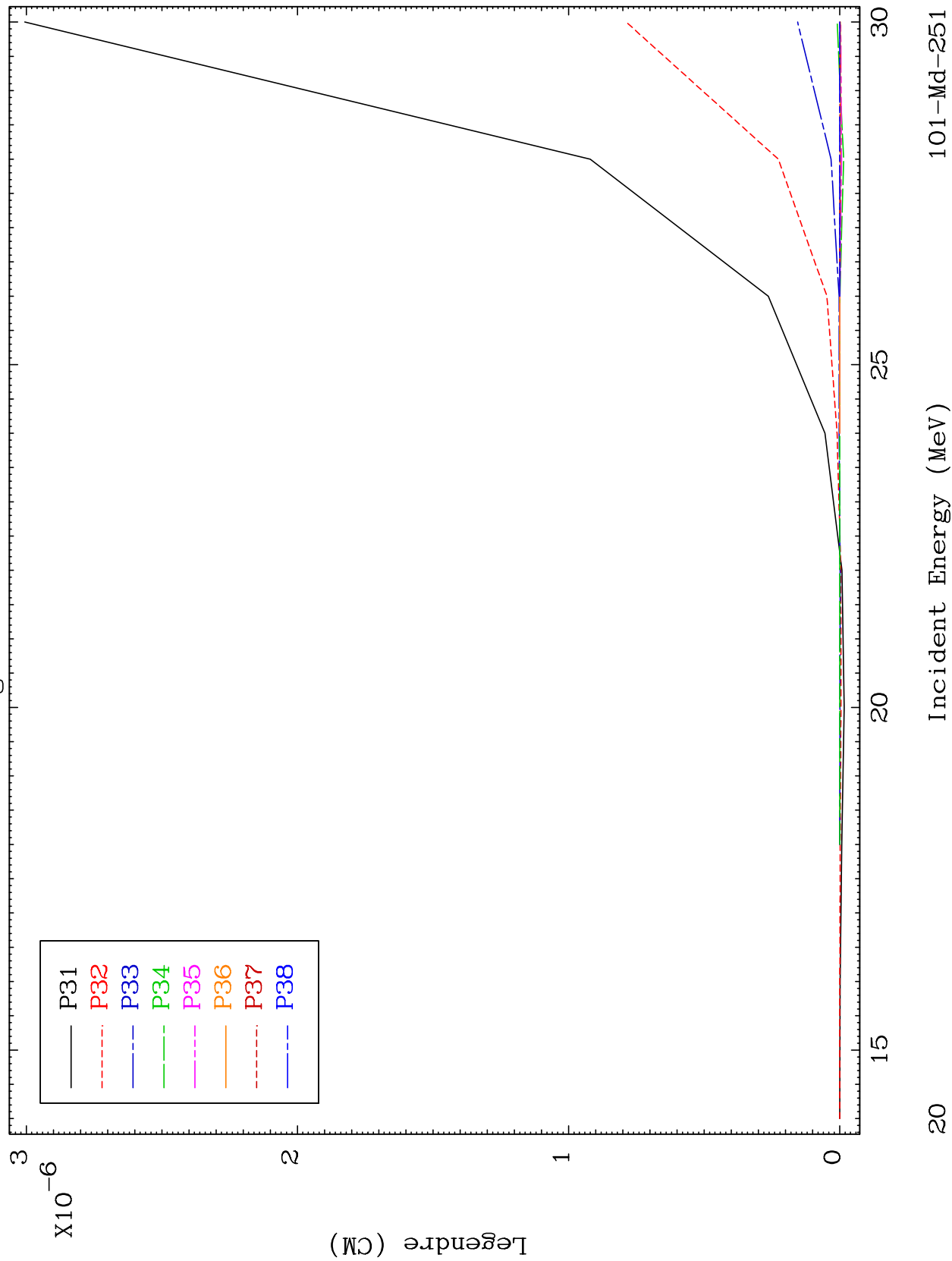
Incident Energy (MeV)

101-Md-251

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Elastic Legendre Coefficients

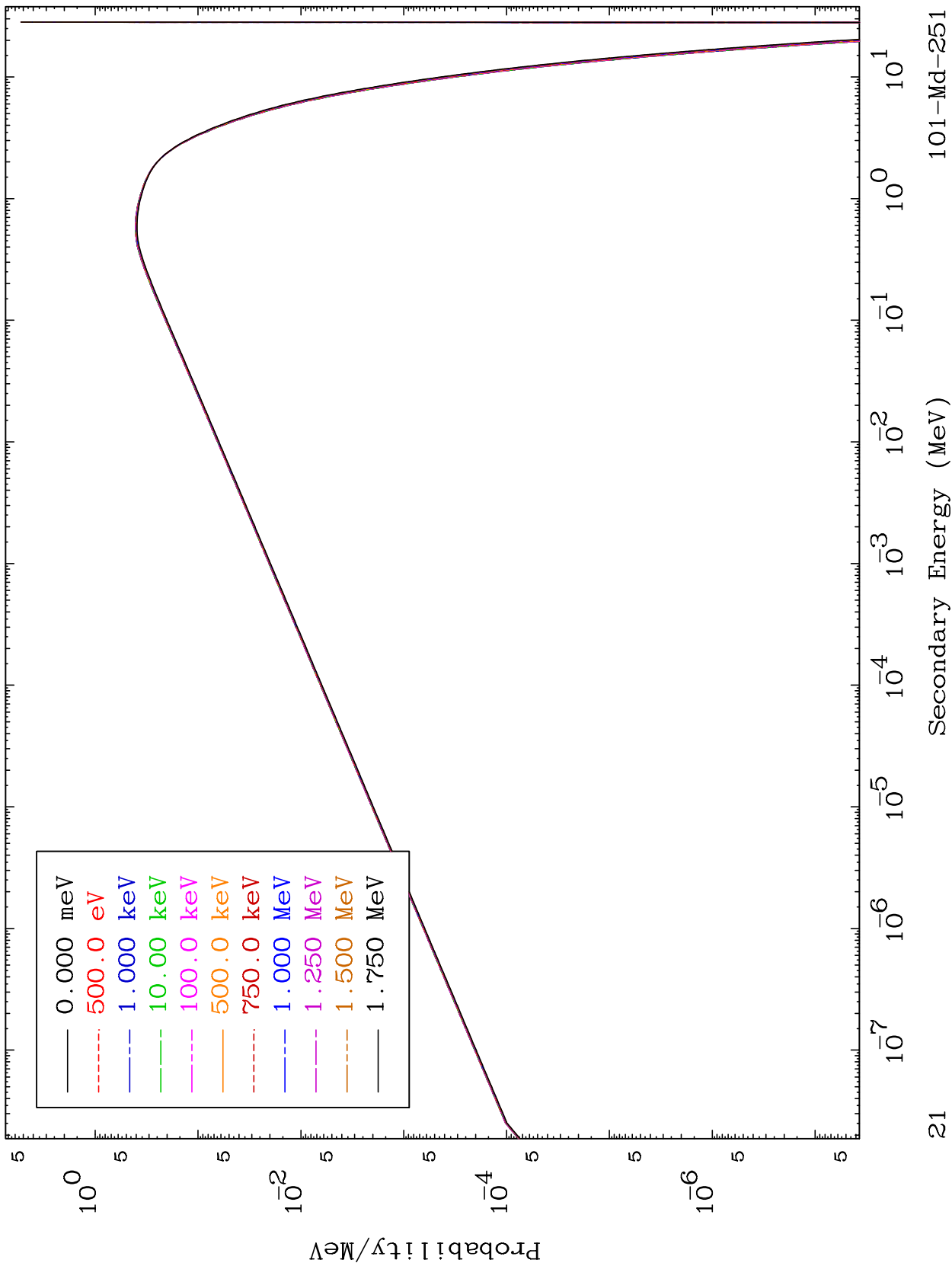
101-Md-251



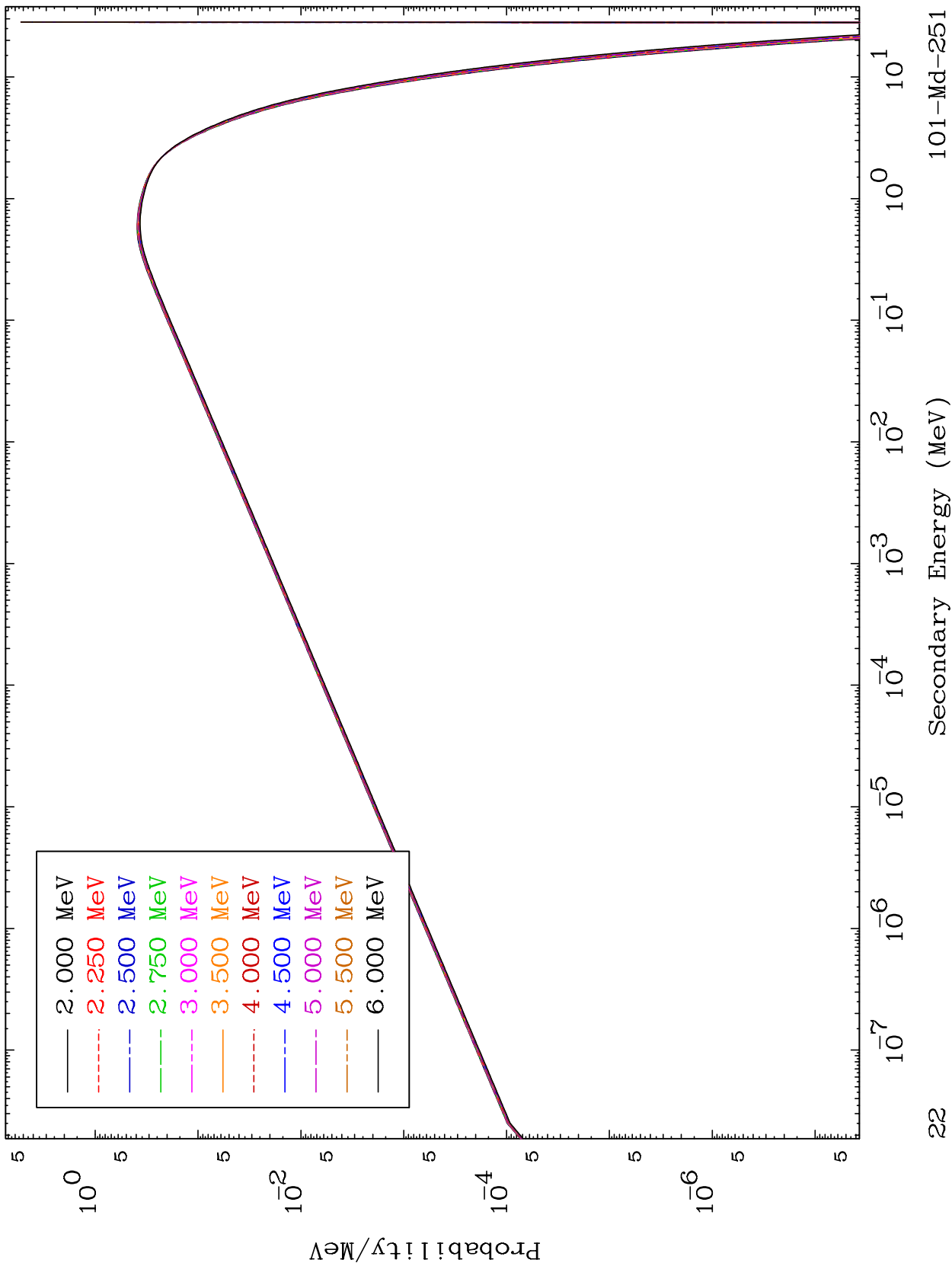
20

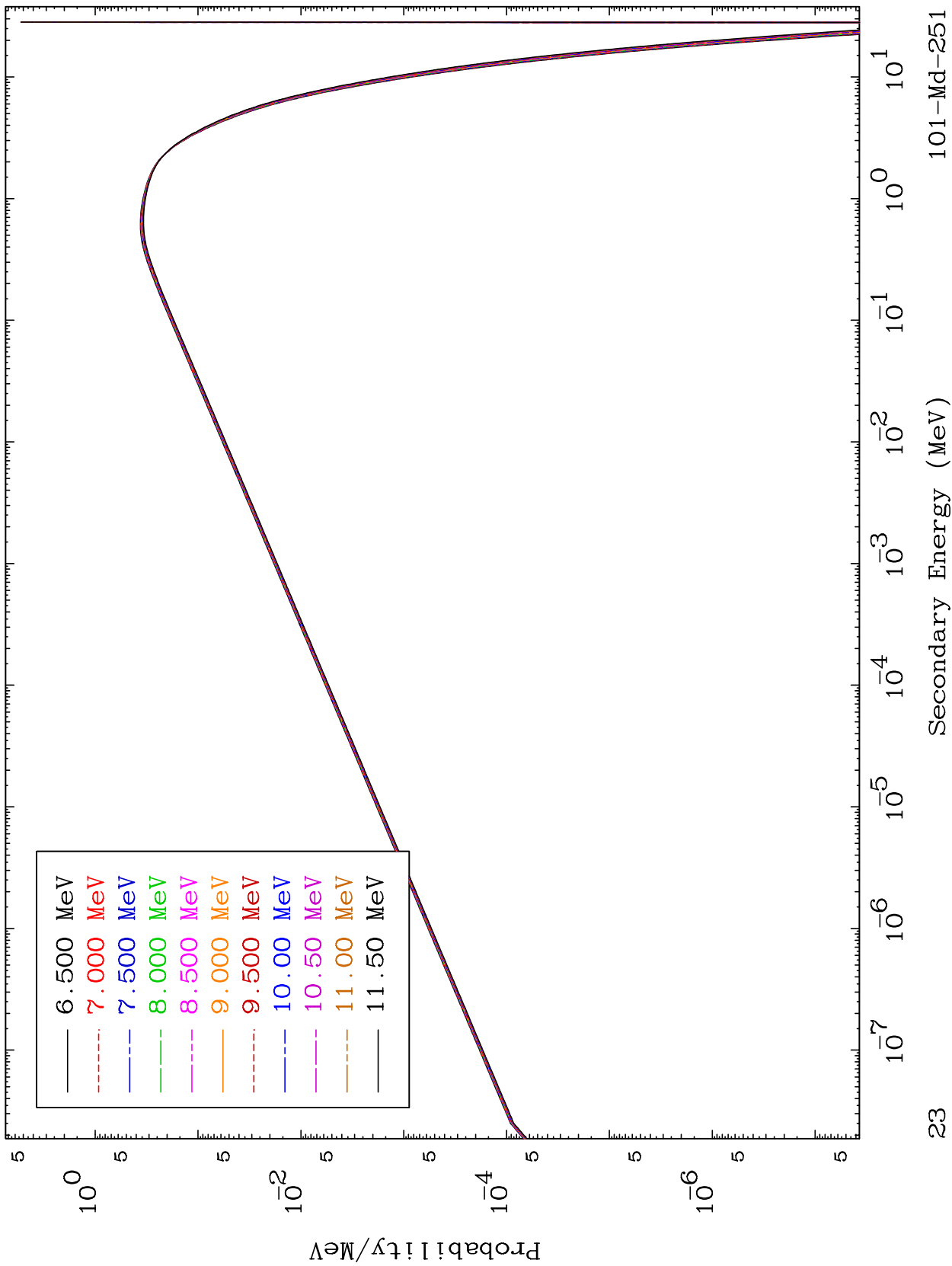
Incident Energy (MeV)

101-Md-251



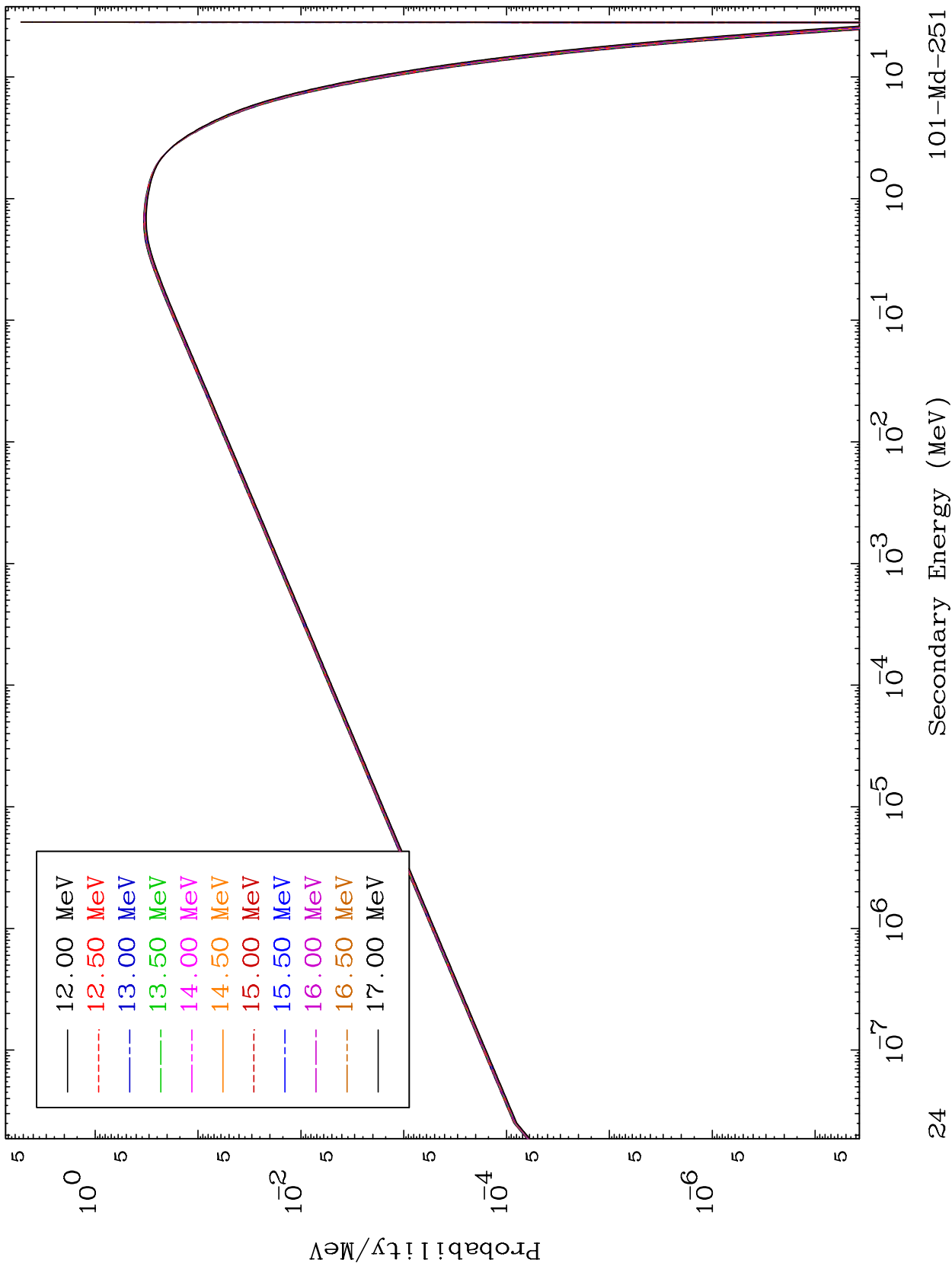
Fission  
Energy Distribution



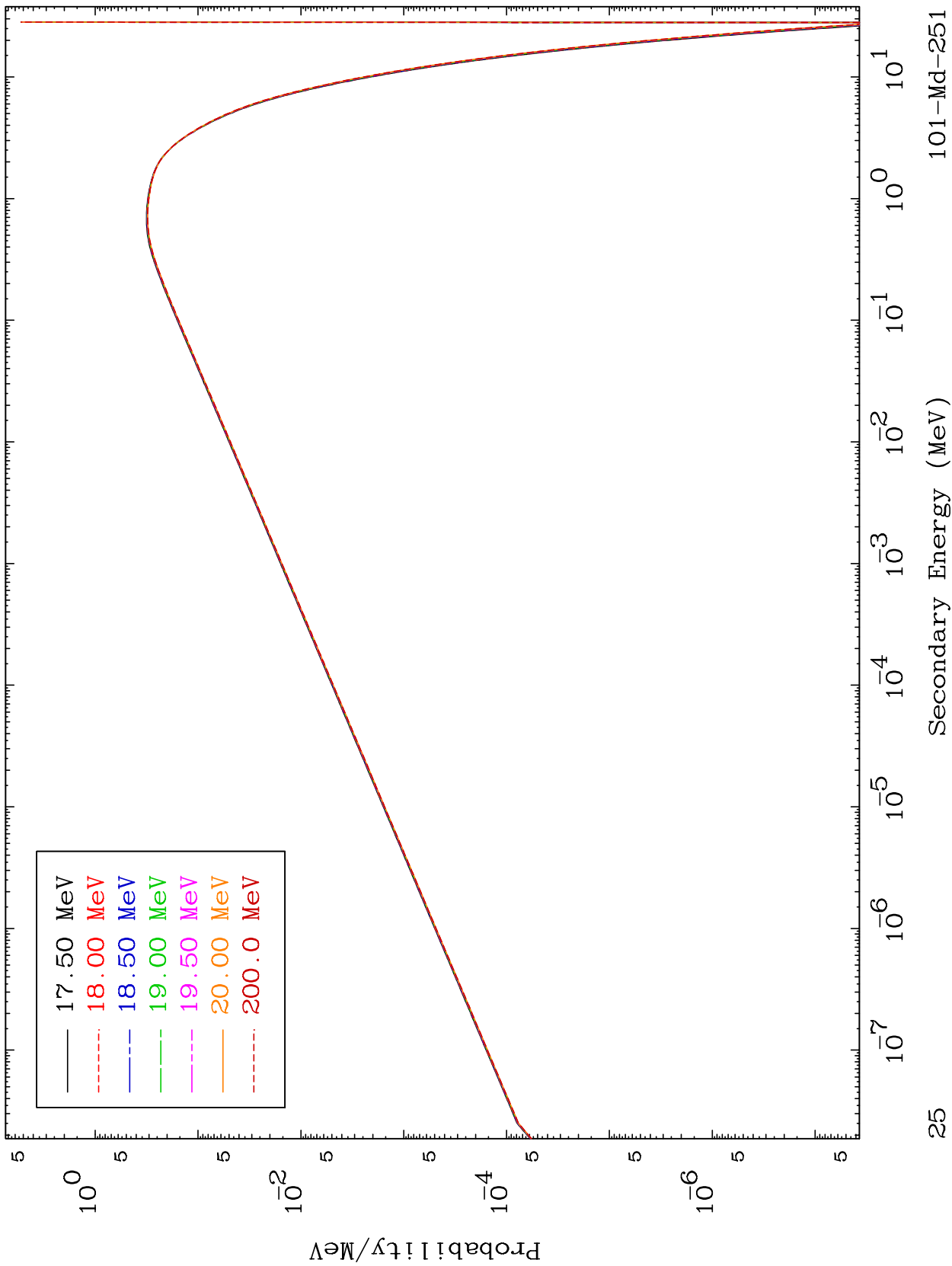




Fission  
Energy Distribution



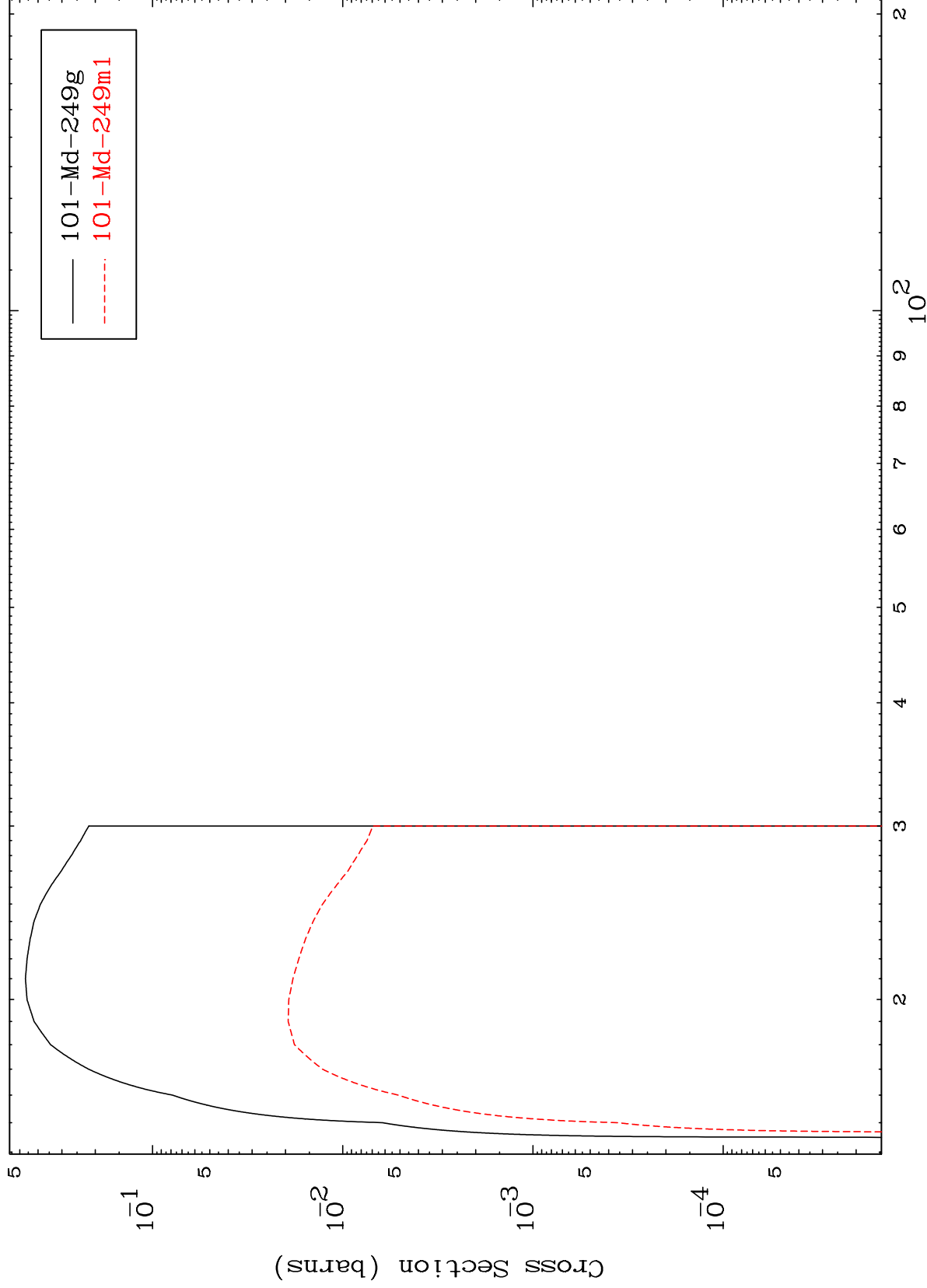
Fission  
Energy Distribution



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101-Md-251

(n,3n)  
Radionuclide Production Cross Section



26

Incident Energy (MeV)

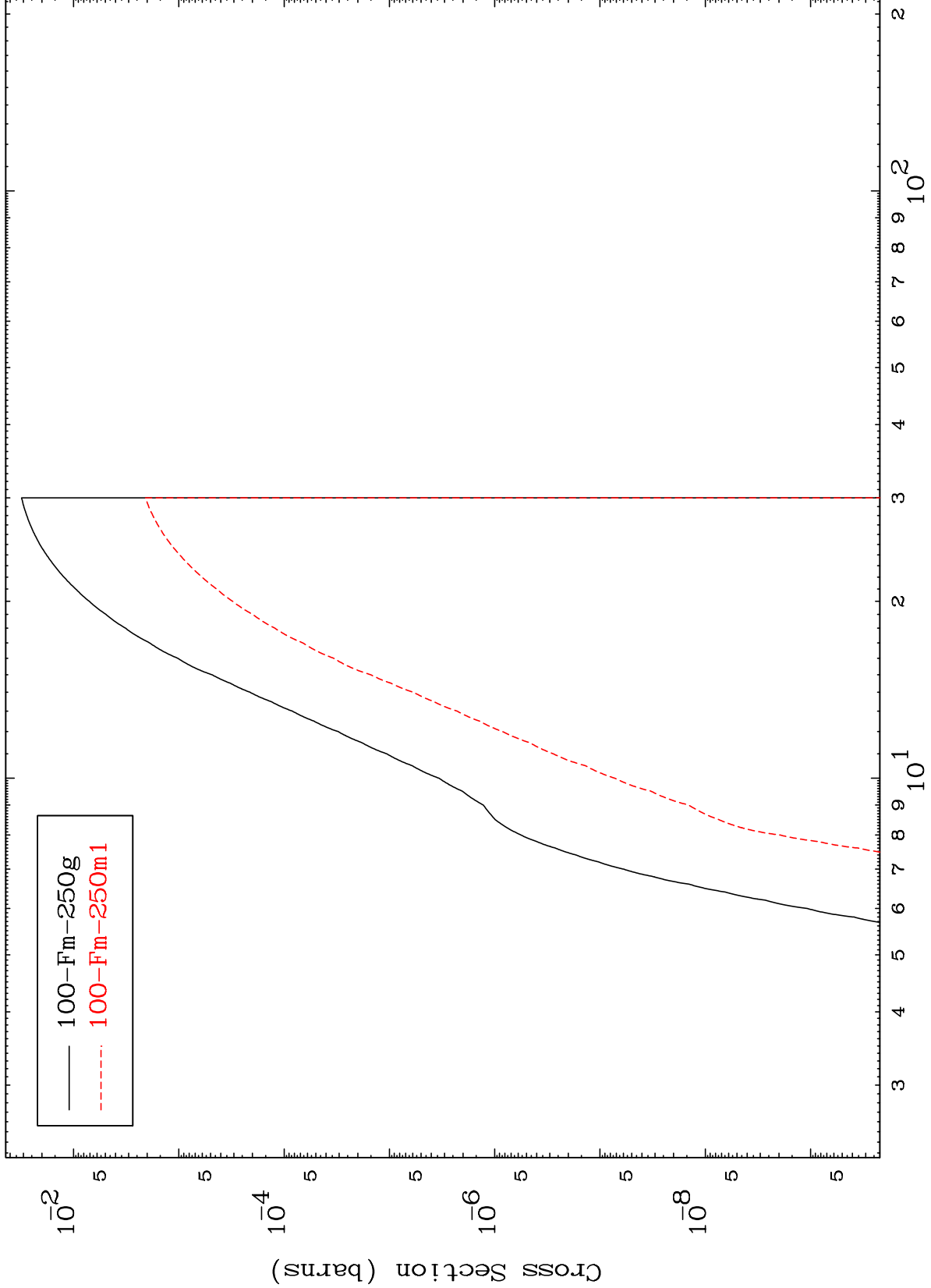
101-Md-251

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(n,n') p

101-Md-251

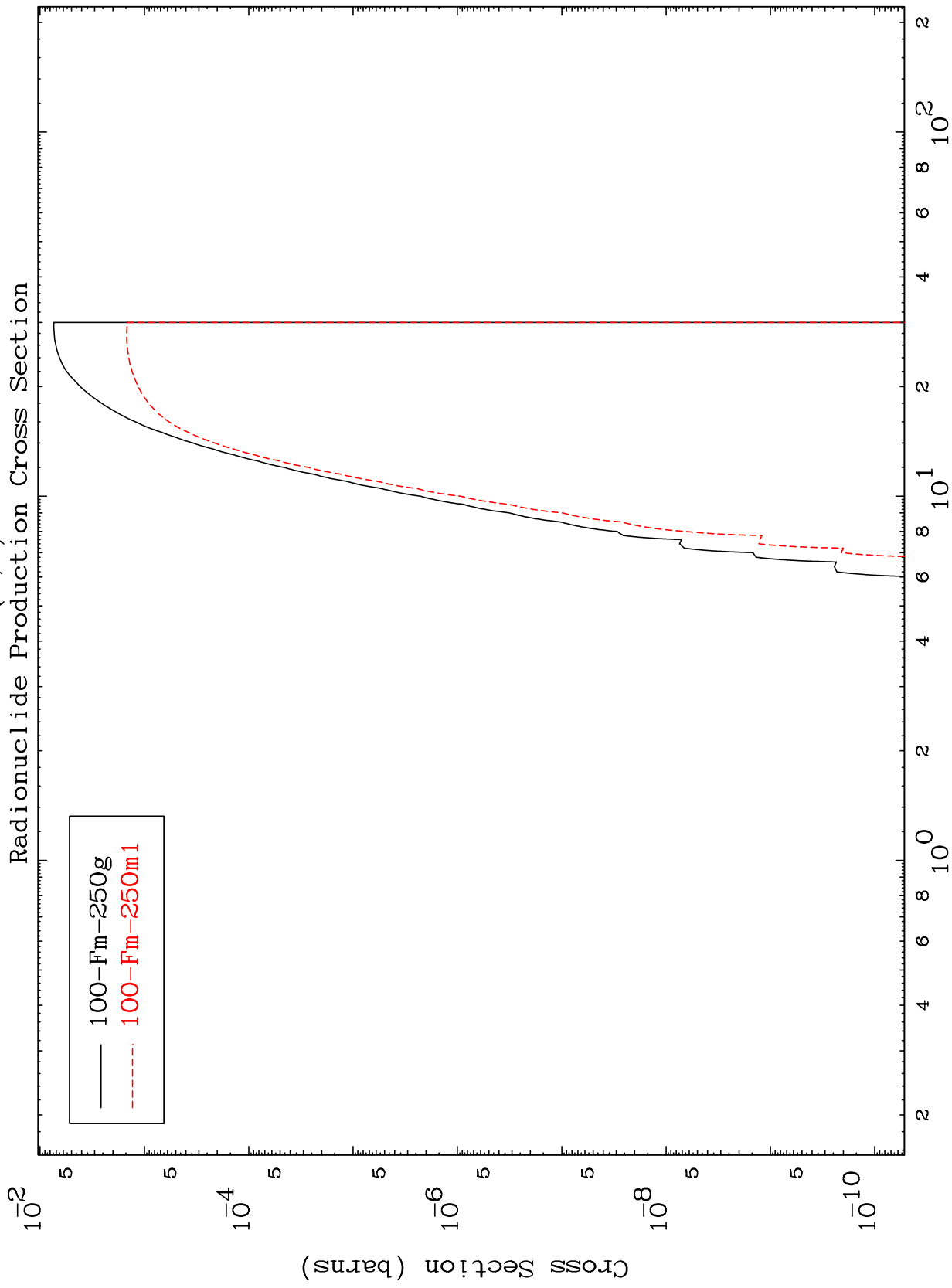
Radionuclide Production Cross Section



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(n, d)

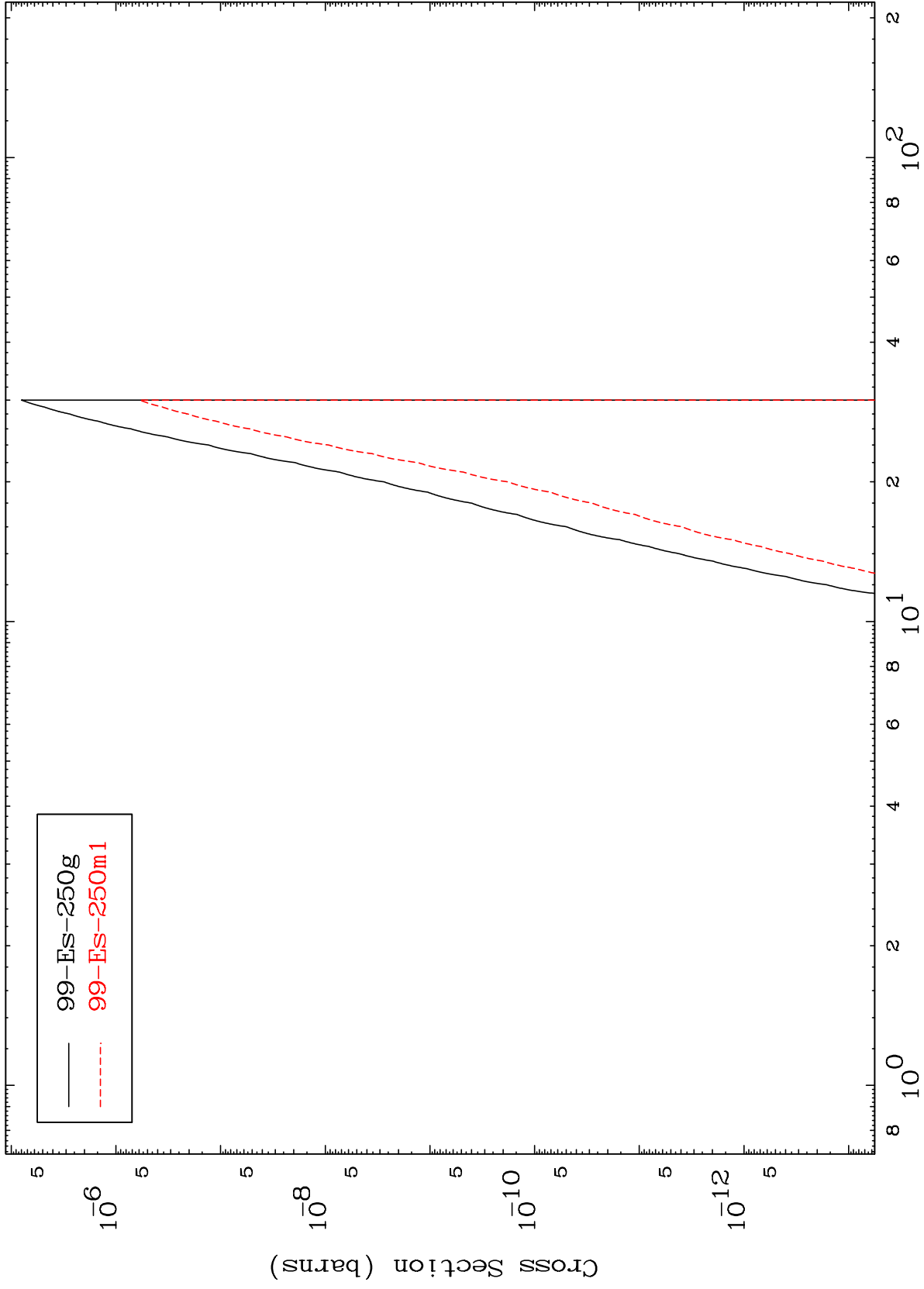
101-Md-251



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101-Md-251

(n,2p)  
Radionuclide Production Cross Section



— 99-Es-250g  
- - - 99-Es-250m1

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Incident Energy (MeV)

101-Md-251