

Program EVALPLOT
(Version 2015-2)

by

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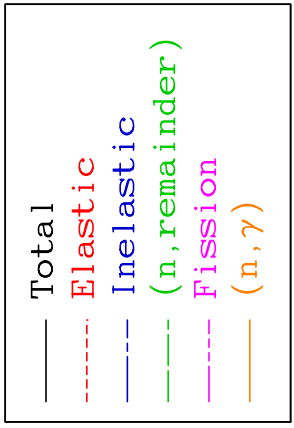
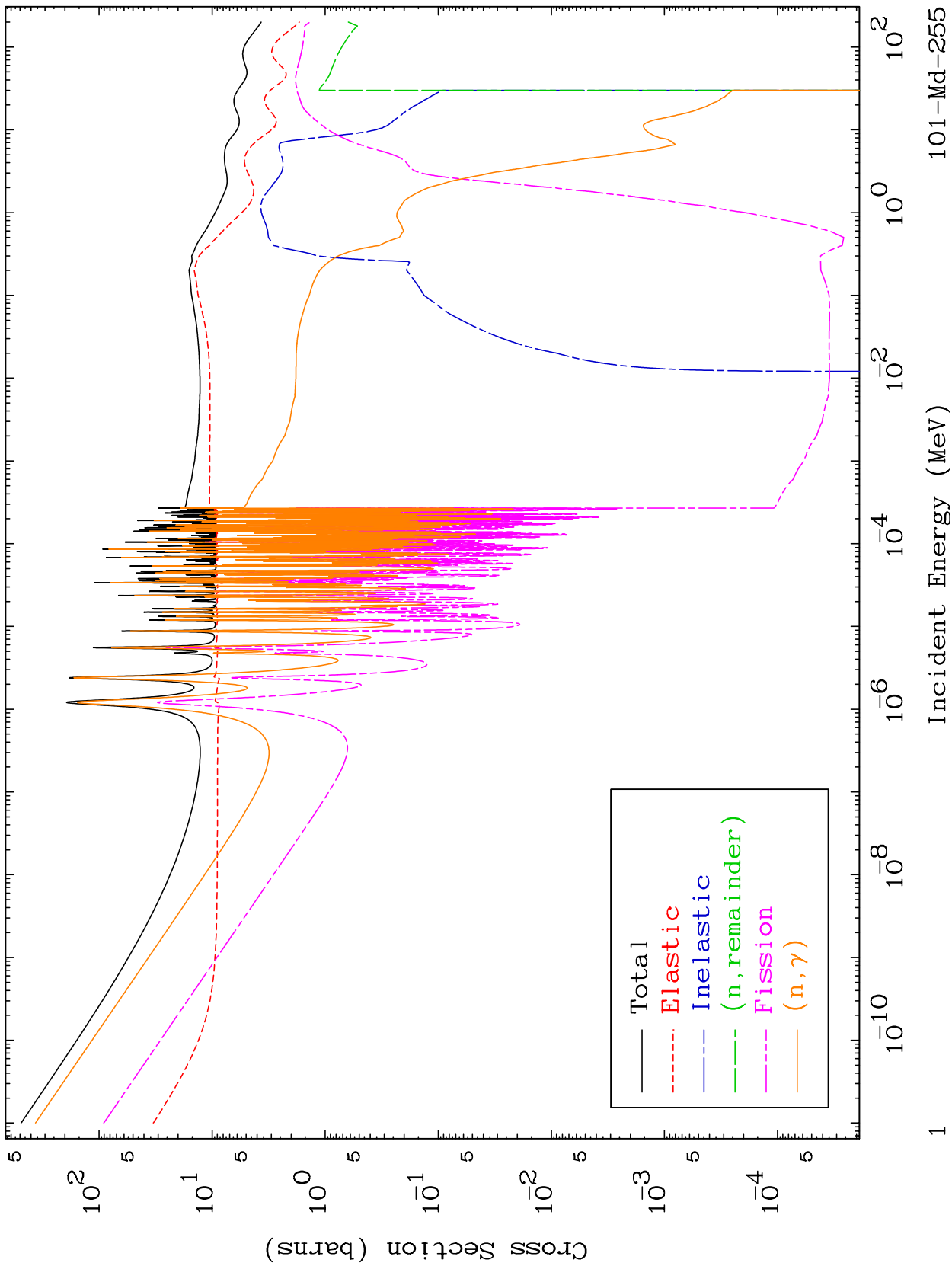
Press Mouse Button to Start

MAT 9956

Major

293 Kelvin Cross Sections

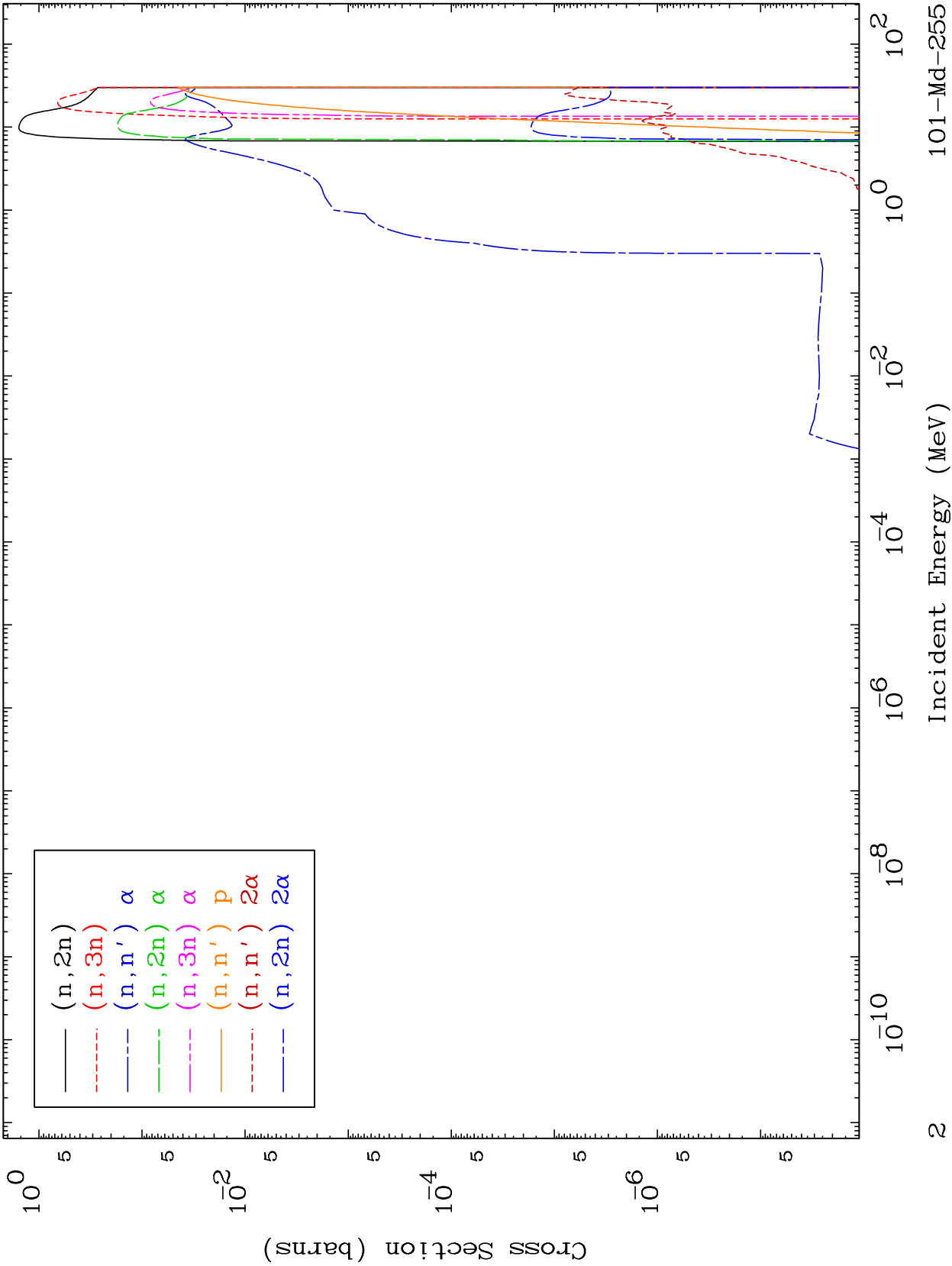
101-Md-255



MAT 9956

Neutron Production
293 Kelvin Cross Sections

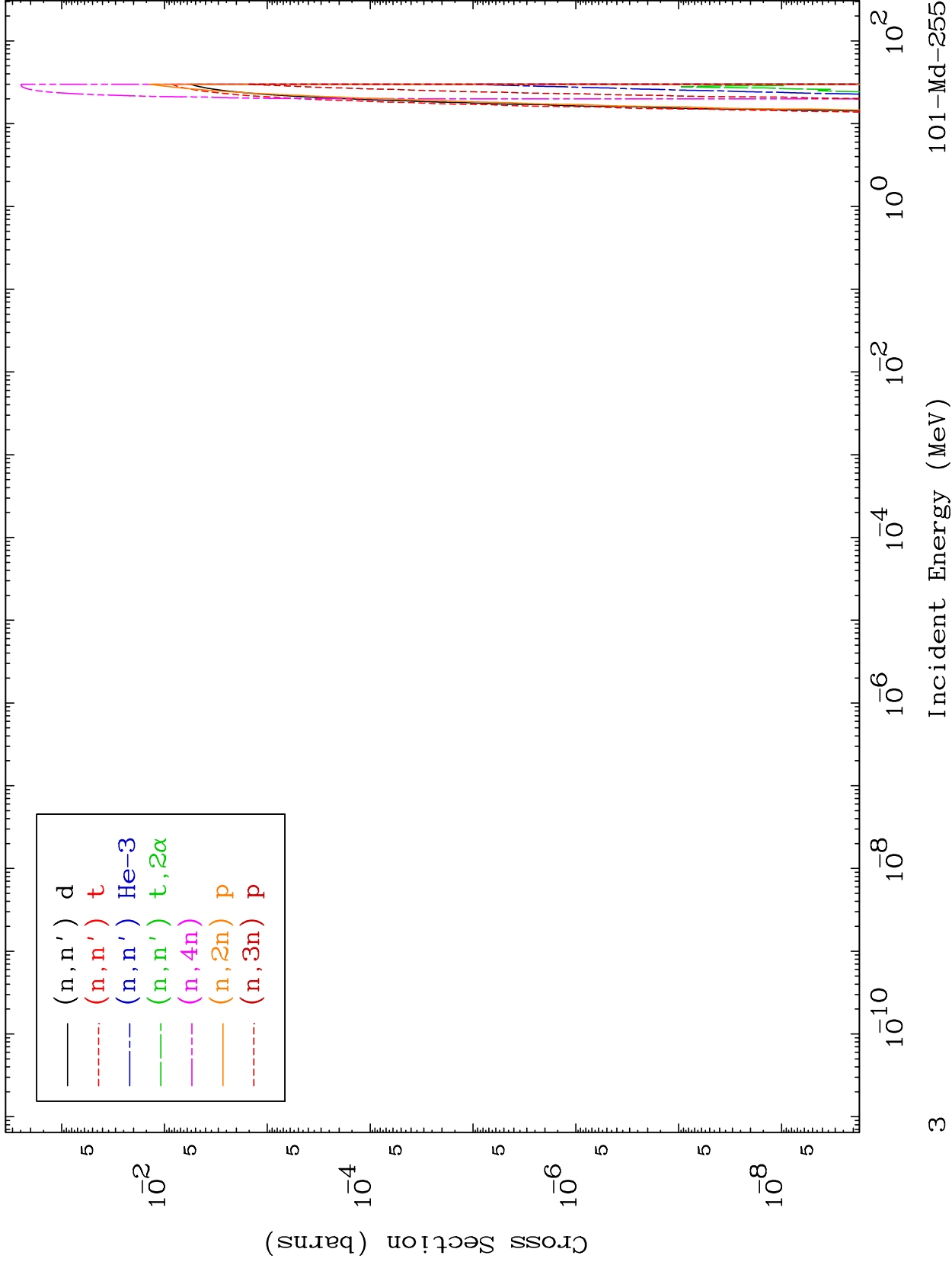
101-Md-255



MAT 9956

Neutron Production
293 Kelvin Cross Sections

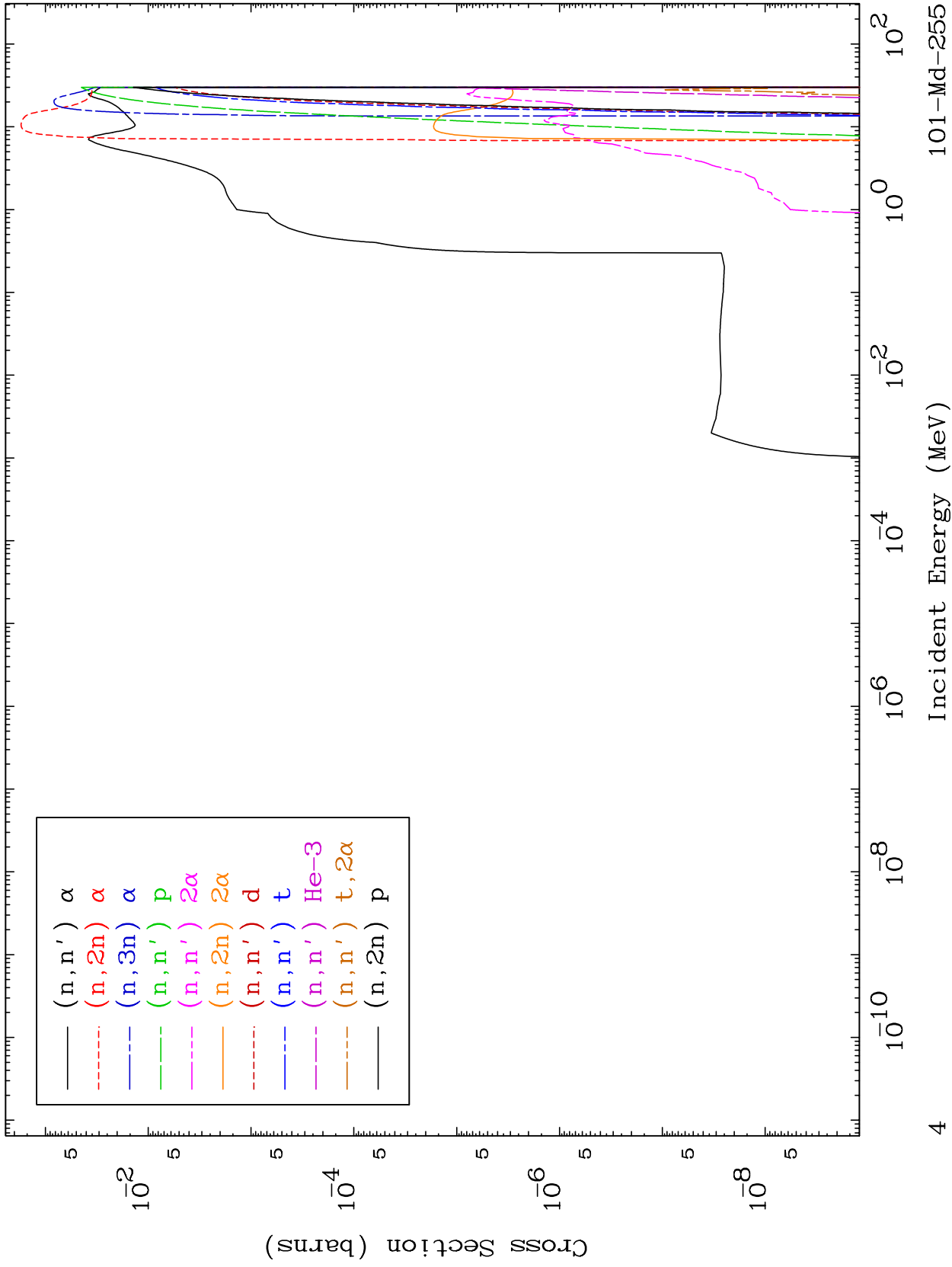
101-Md-255



MAT 9956

Charged Particle
293 Kelvin Cross Sections

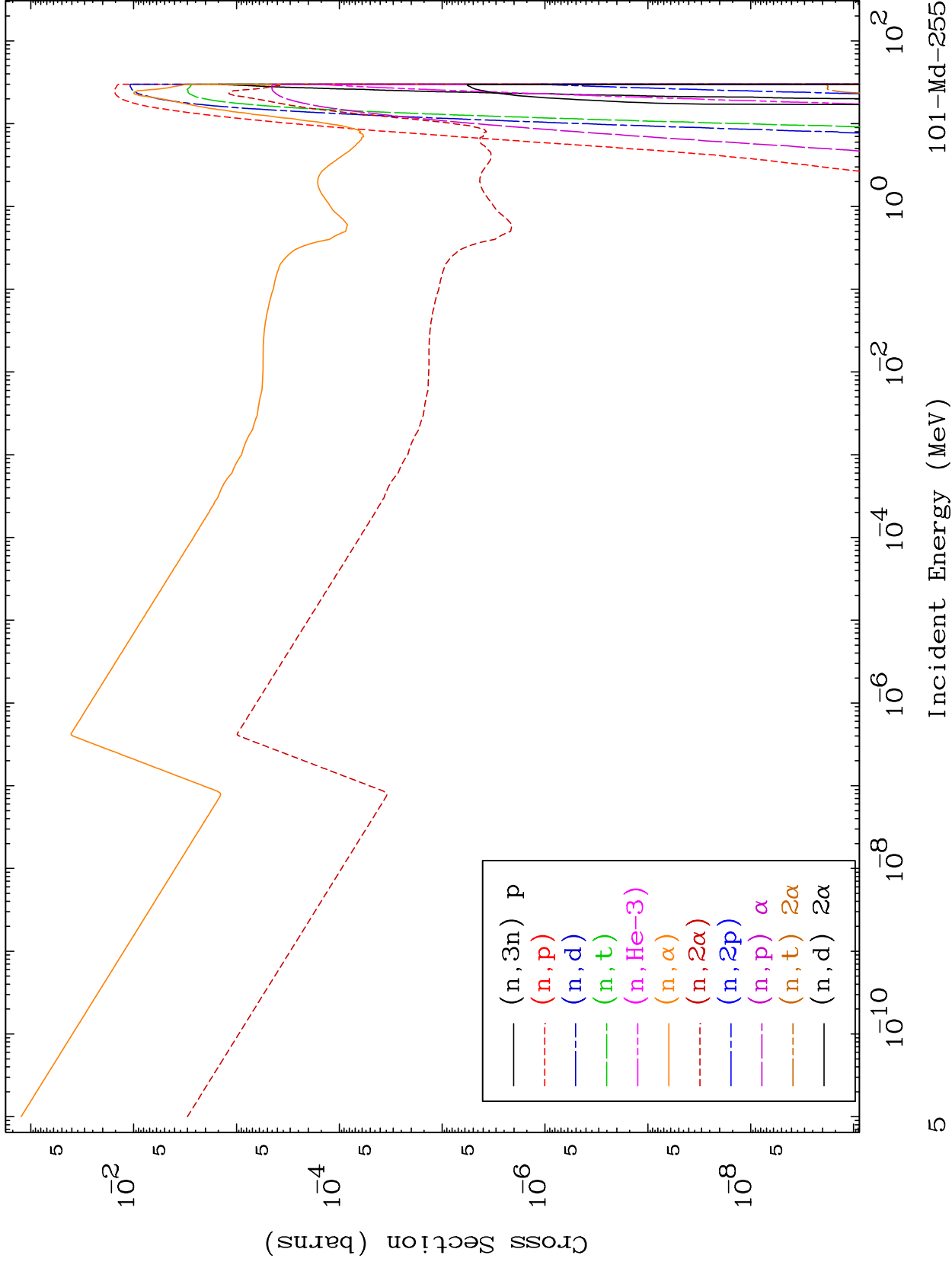
101-Md-255



MAT 9956

Charged Particle
293 Kelvin Cross Sections

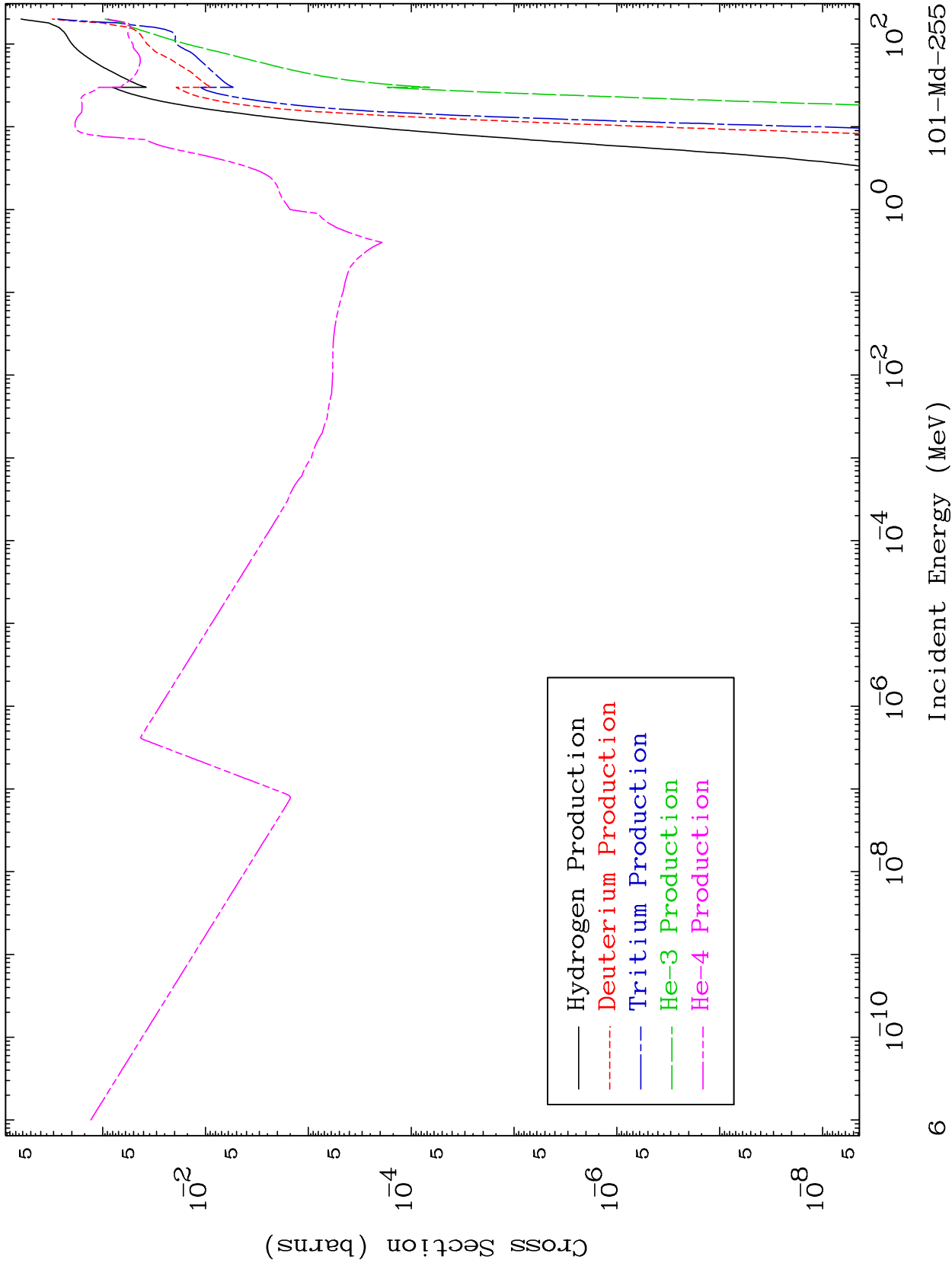
101-Md-255



MAT 9956

Particle Production
293 Kelvin Cross Sections

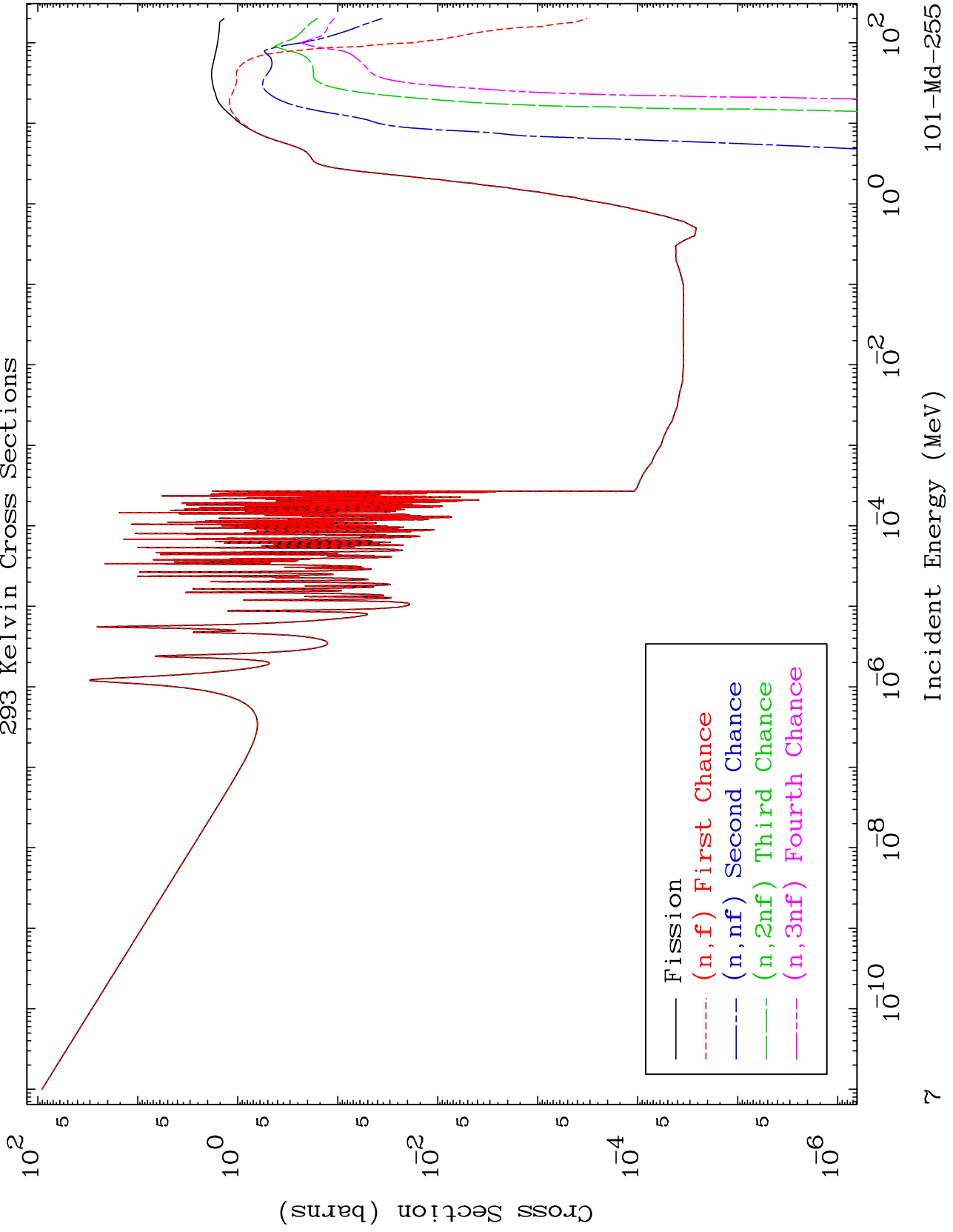
101-Md-255



MAT 9956

Fission
293 Kelvin Cross Sections

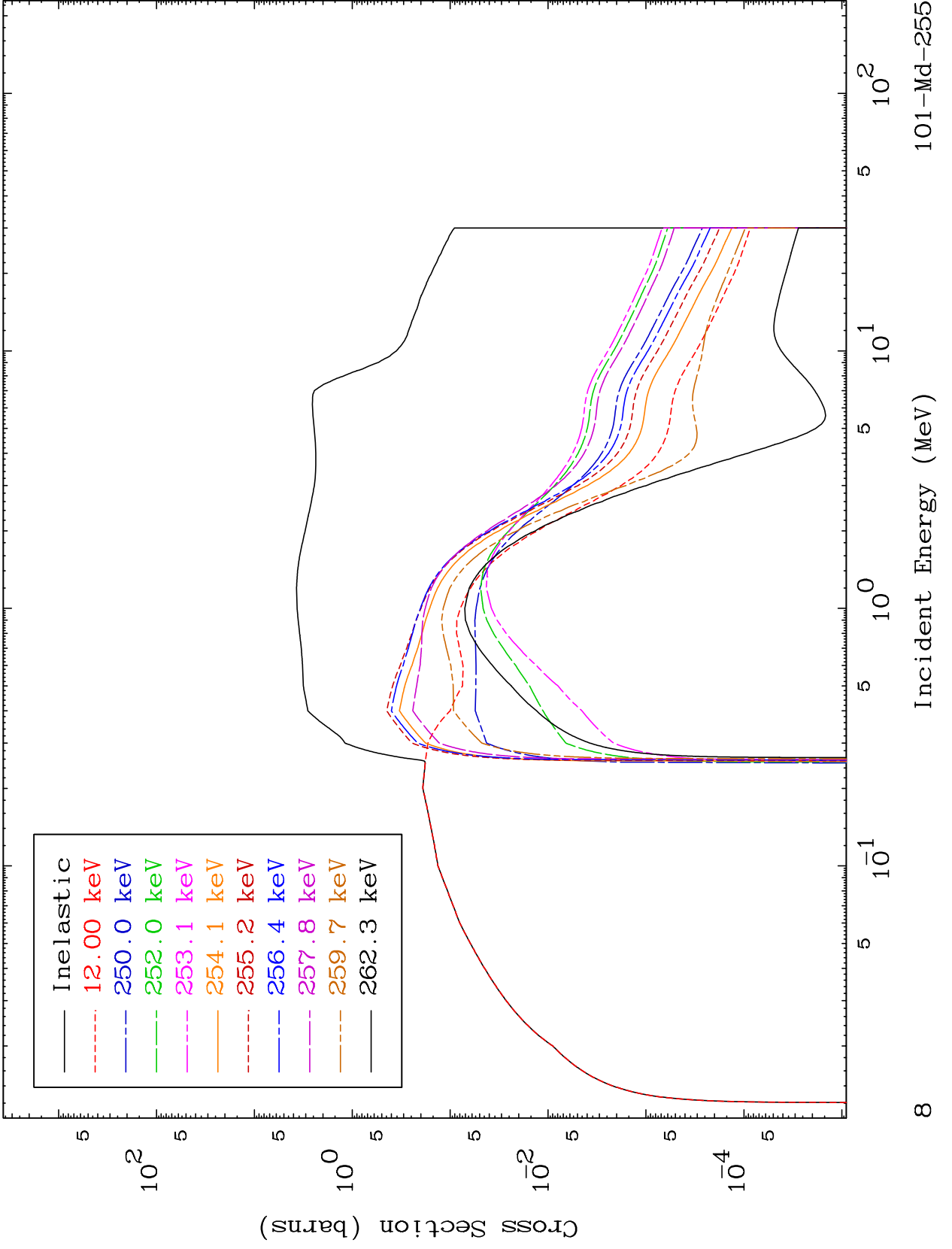
101-Md-255



MAT 9956

(n,n') Level
293 Kelvin Cross Sections

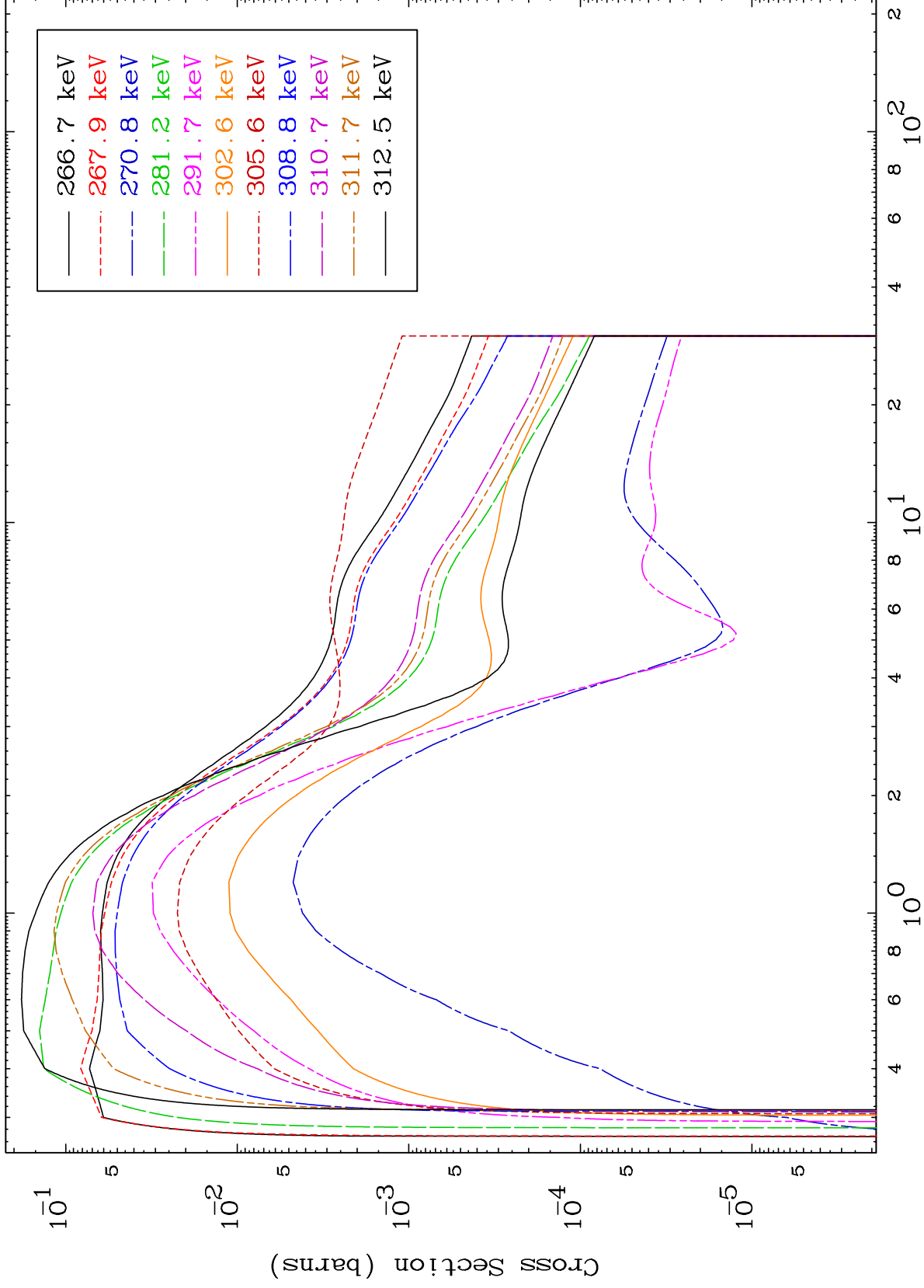
101-Md-255



MAT 9956

(n,n') Level
293 Kelvin Cross Sections

101-Md-255



9

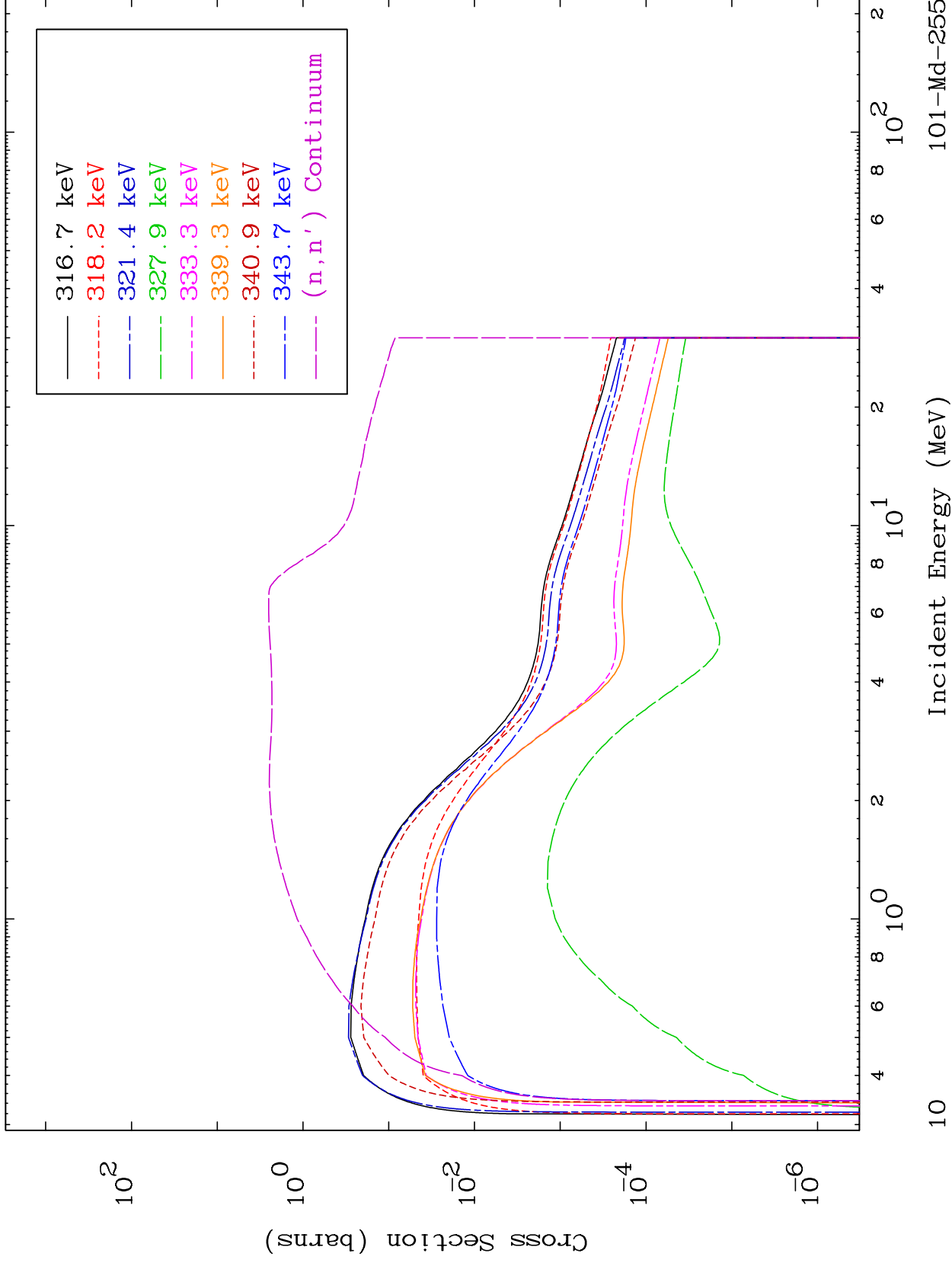
Incident Energy (MeV)

101-Md-255

MAT 9956

(n,n') Level
293 Kelvin Cross Sections

101-Md-255



10

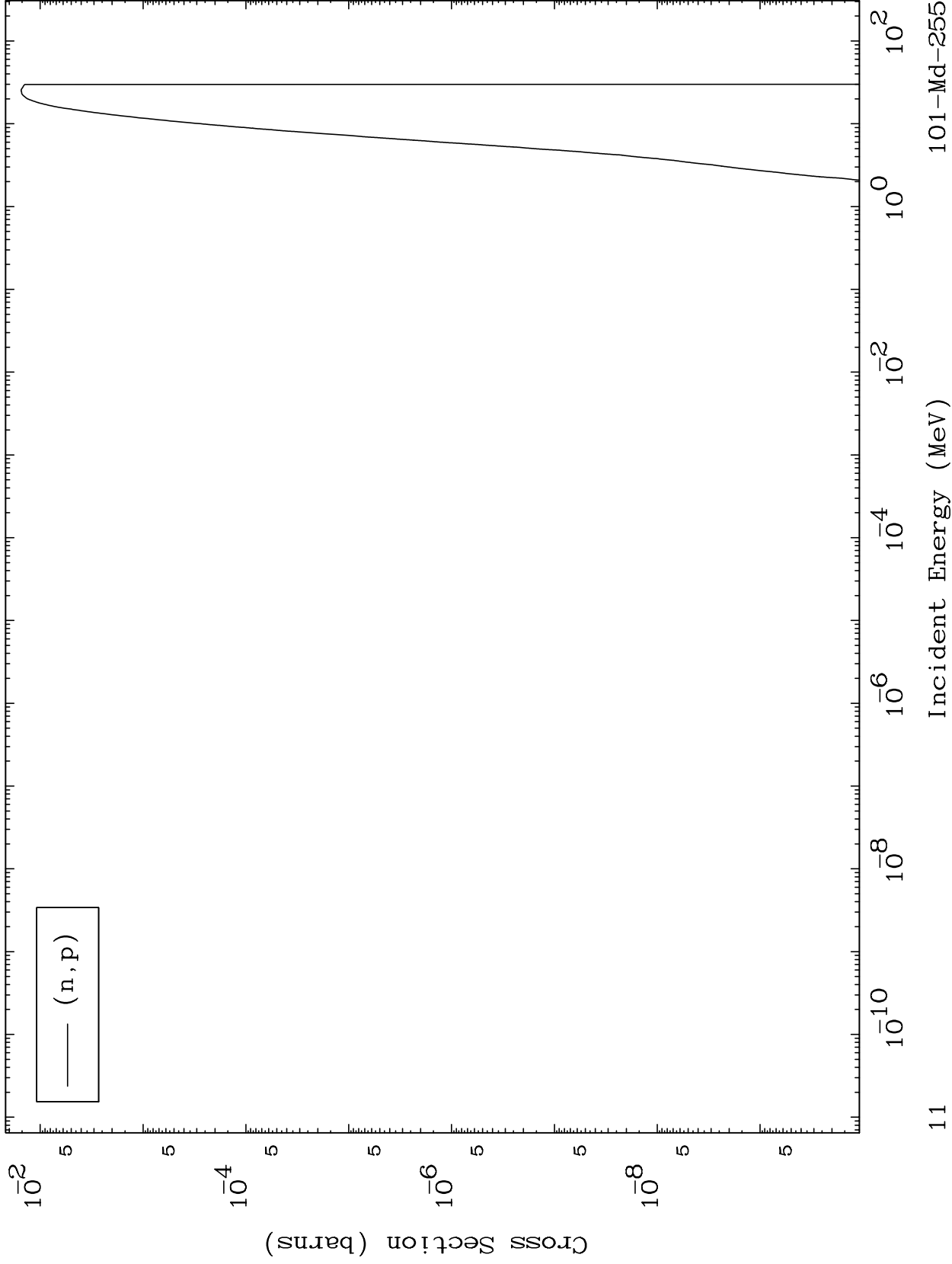
Incident Energy (MeV)

101-Md-255

MAT 9956

(n,p) Levels
293 Kelvin Cross Sections

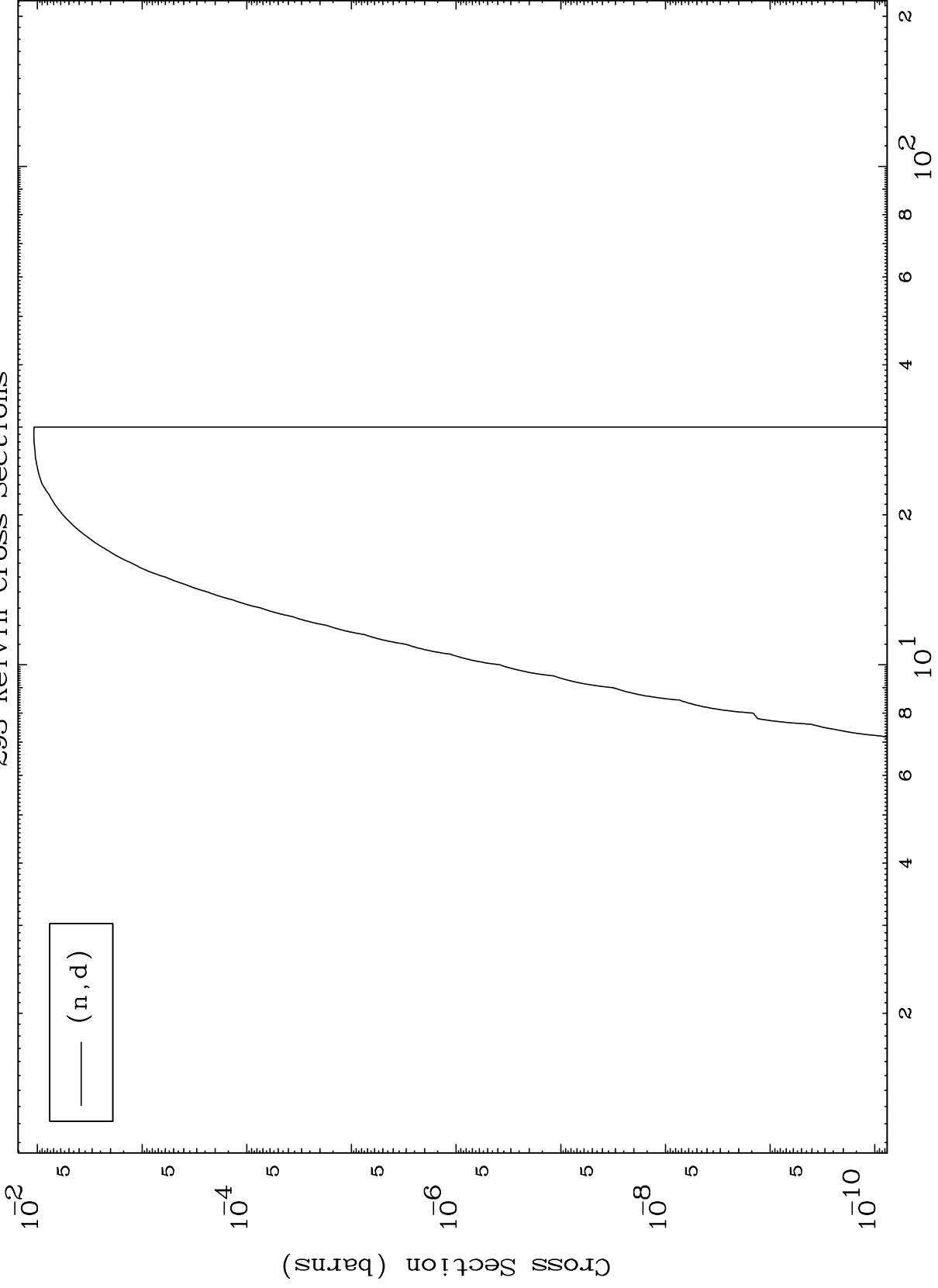
101-Md-255



MAT 9956

(n,d) Levels
293 Kelvin Cross Sections

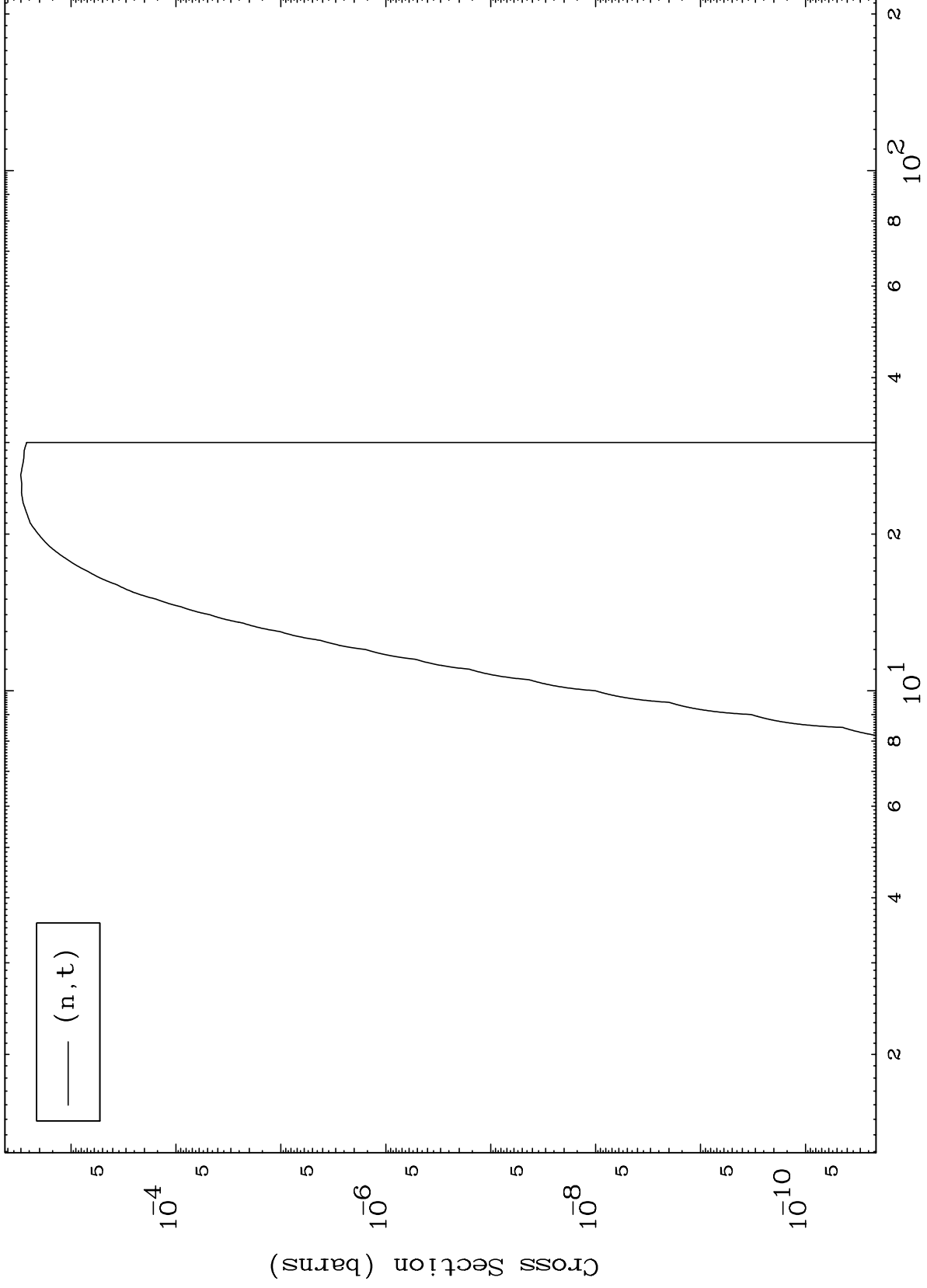
101-Md-255



MAT 9956

(n,t) Levels
293 Kelvin Cross Sections

101-Md-255



13

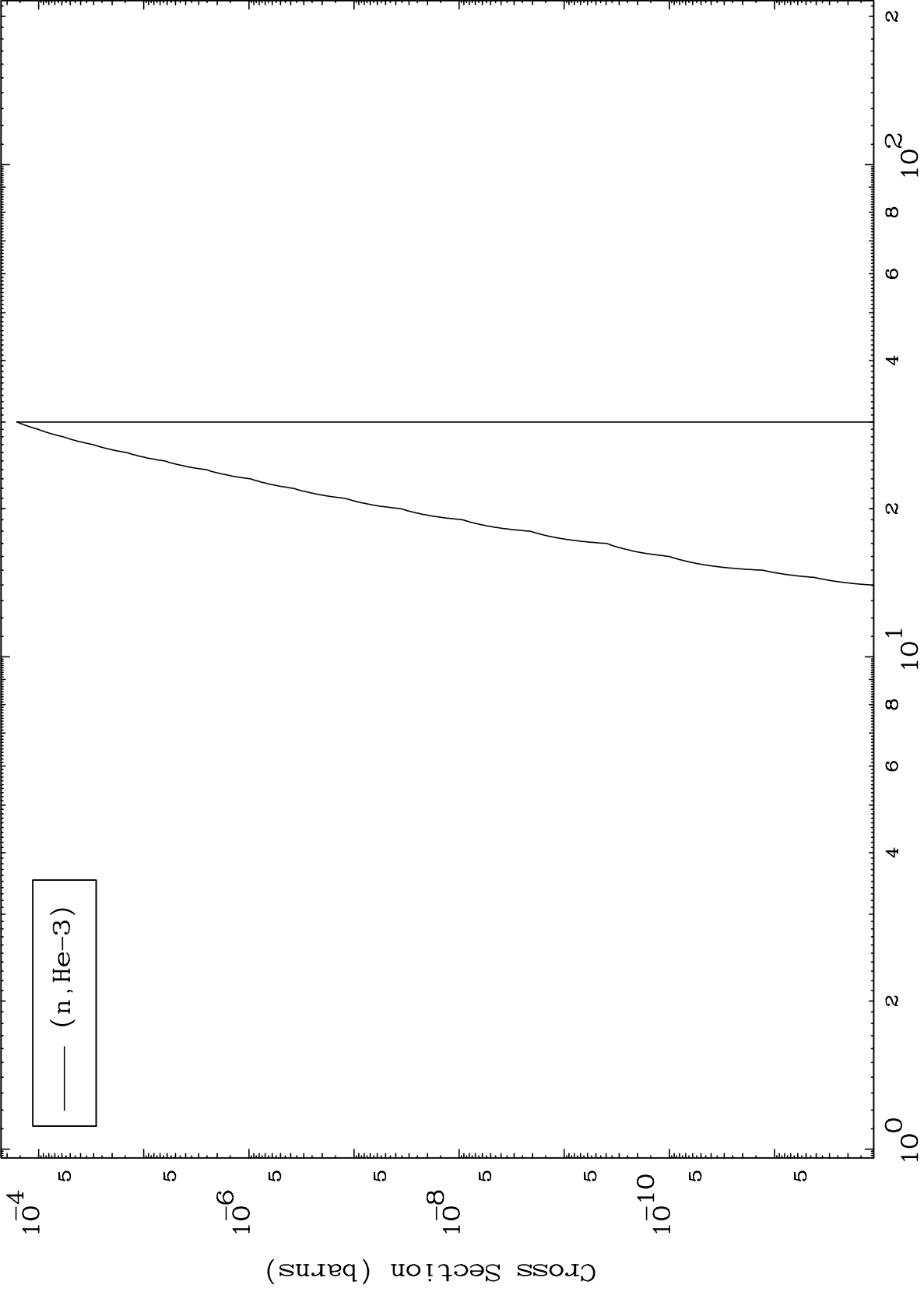
Incident Energy (MeV)

101-Md-255

MAT 9956

(n,He3) Levels
293 Kelvin Cross Sections

101-Md-255



14

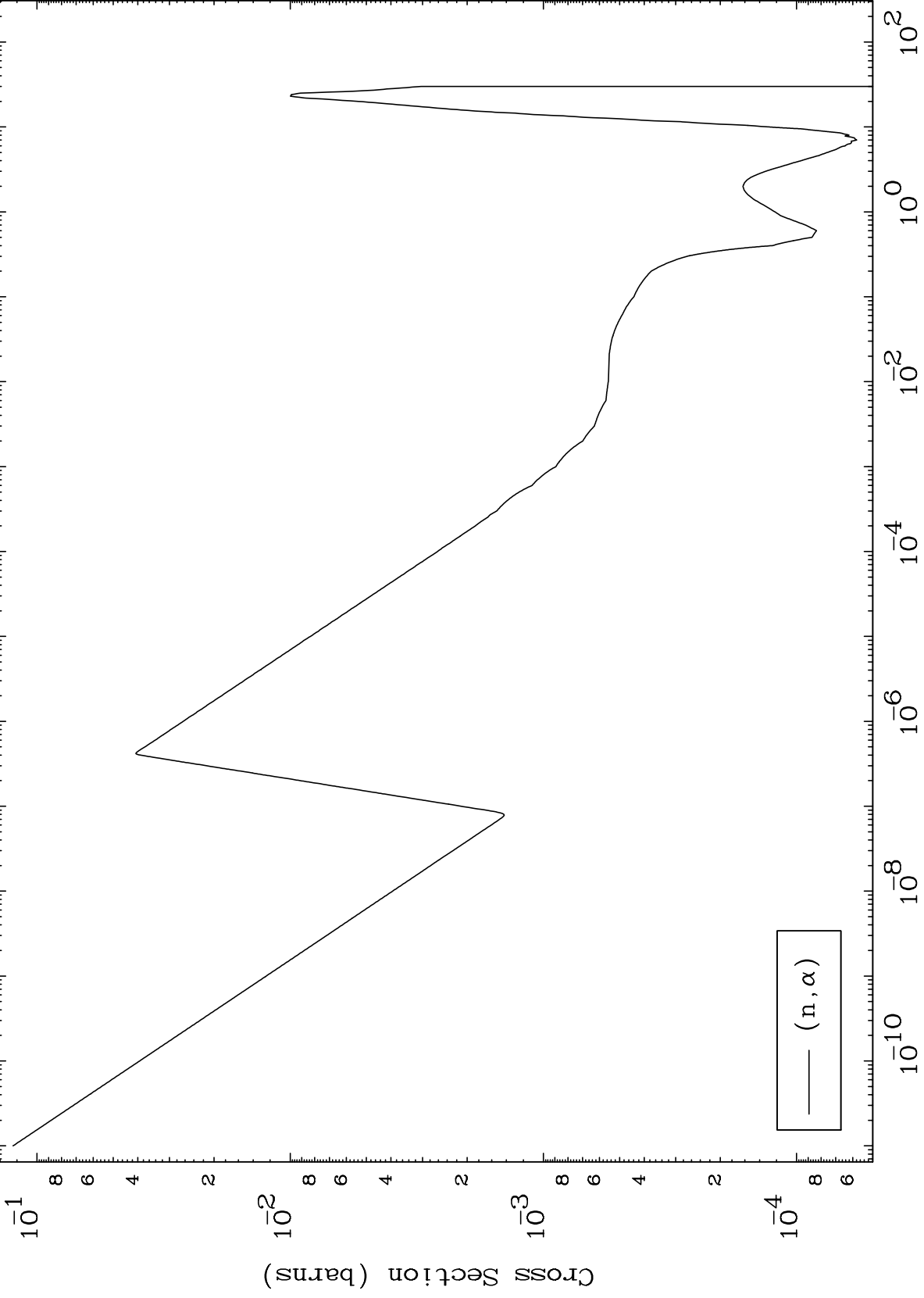
Incident Energy (MeV)

101-Md-255

MAT 9956

(n,α) Levels
293 Kelvin Cross Sections

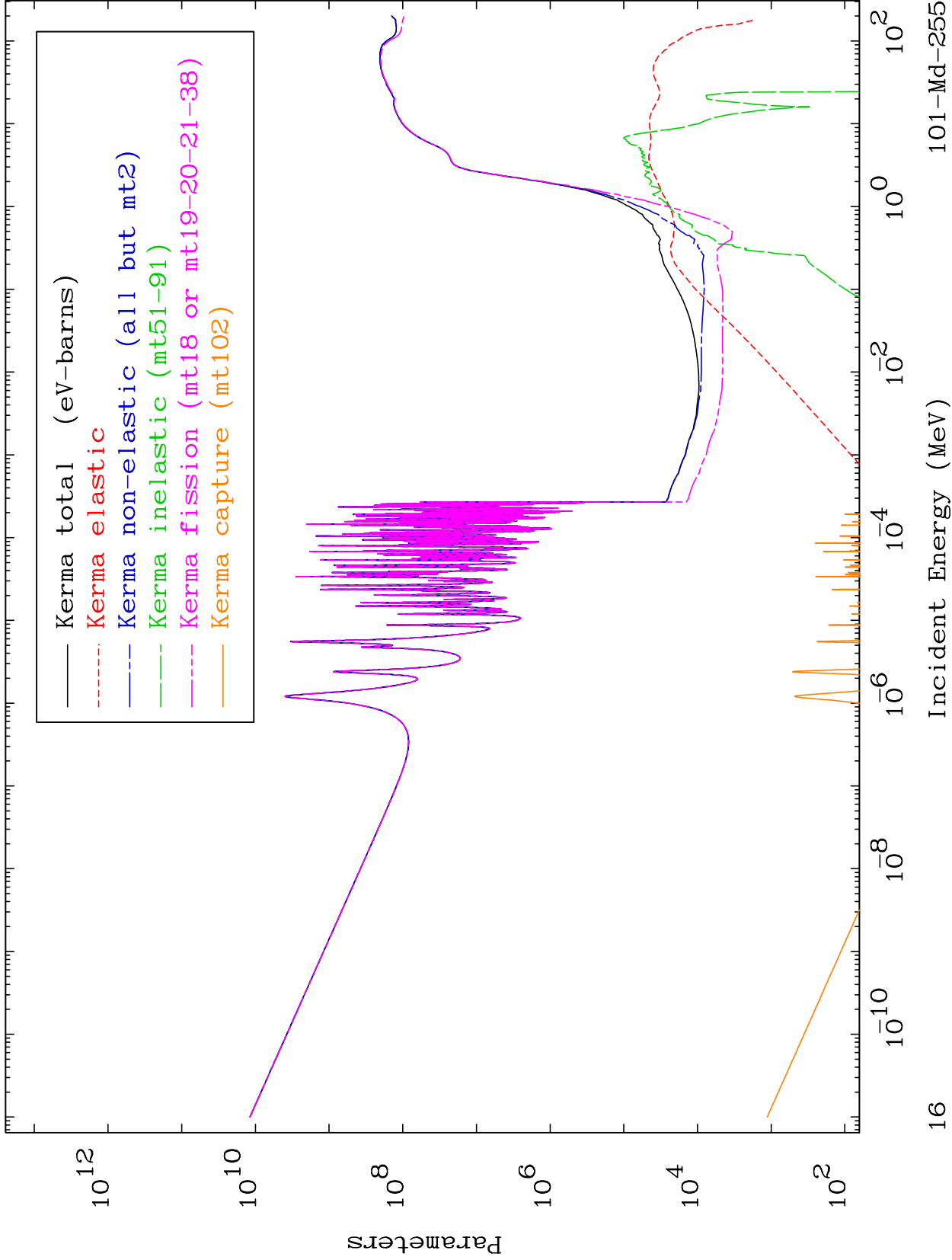
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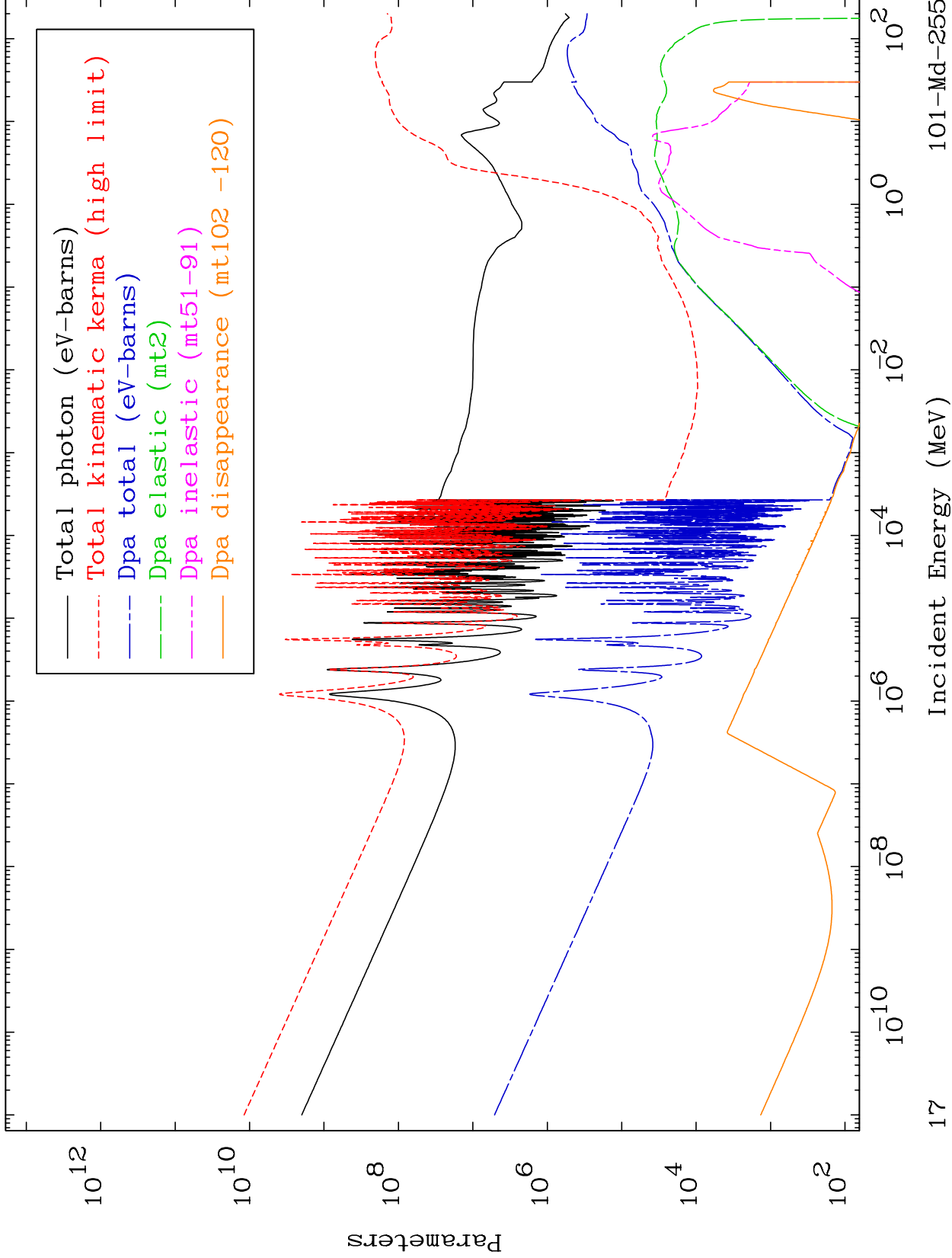


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Incident Energy (MeV)

101-Md-255

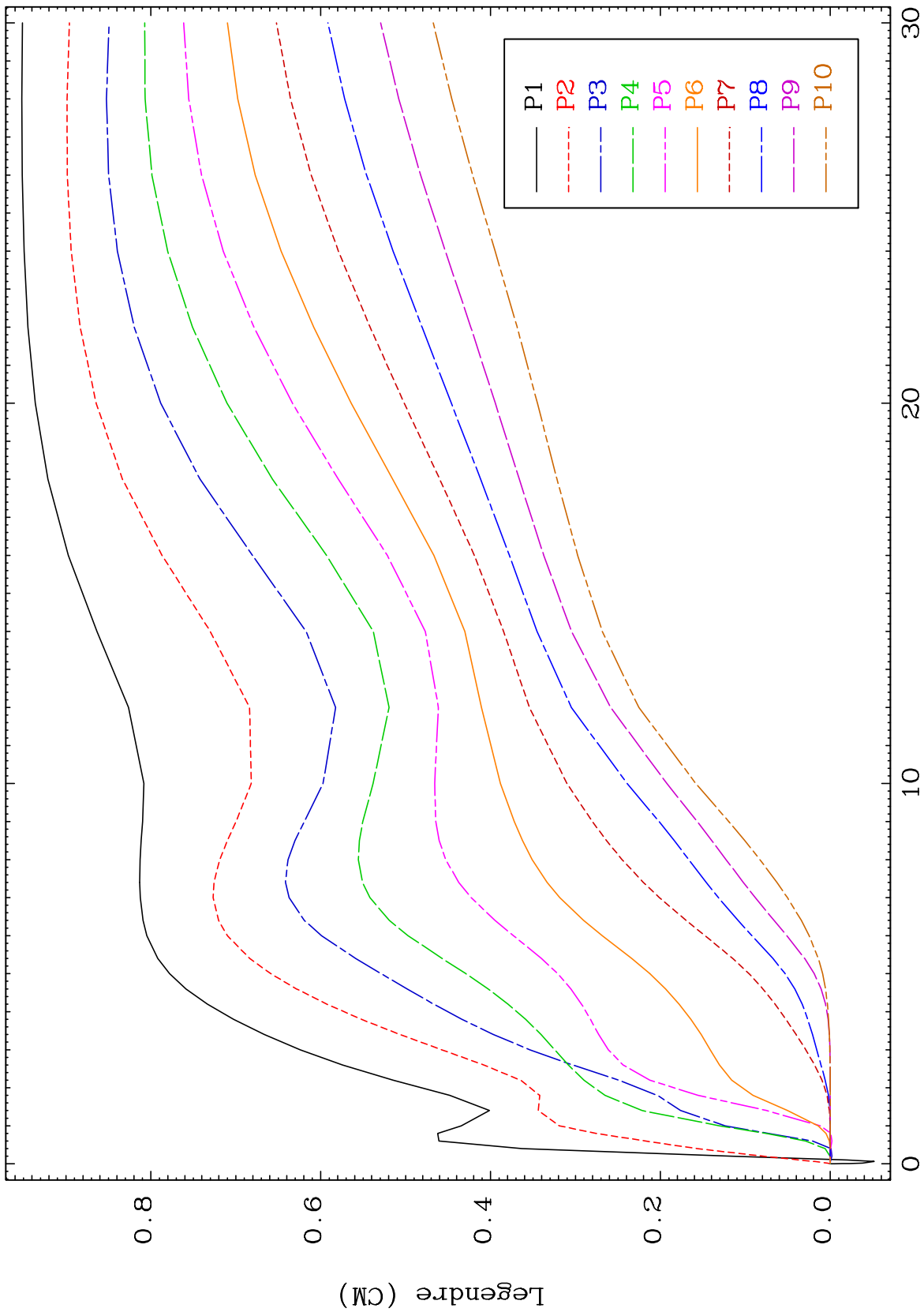




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Elastic Legendre Coefficients

101-Md-255



18

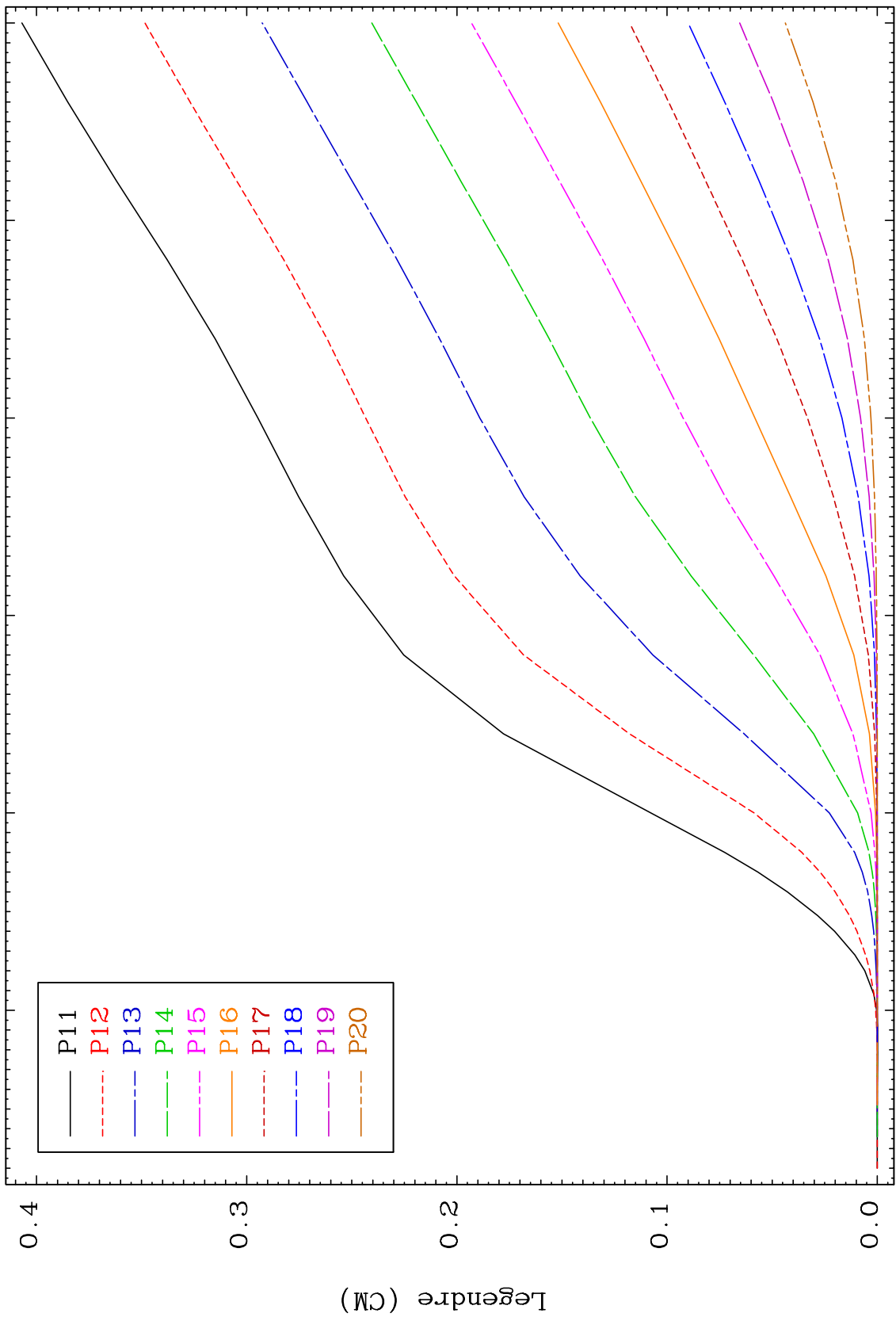
Incident Energy (MeV)

101-Md-255

MAT 9956

Elastic Legendre Coefficients

101-Md-255



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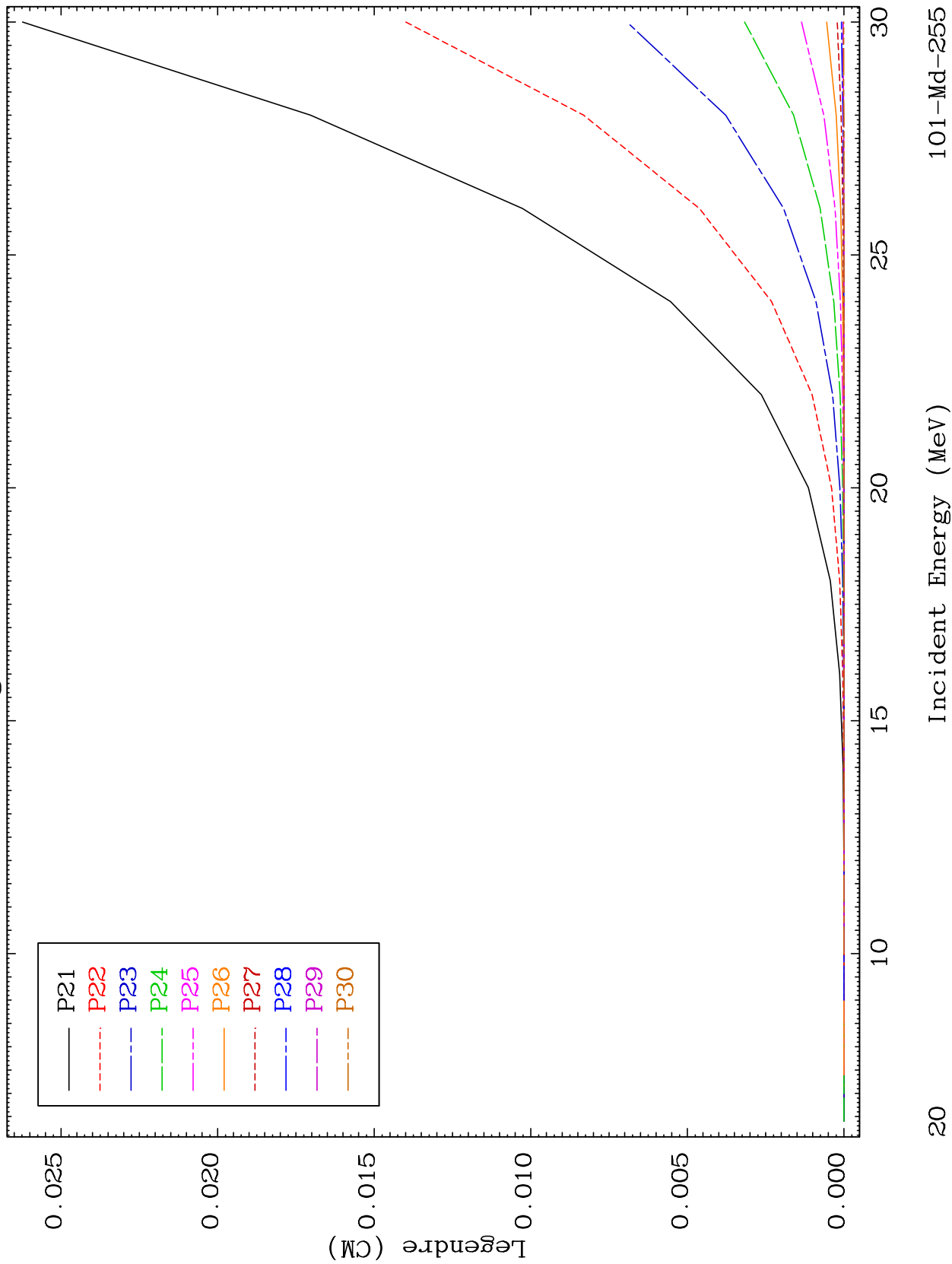
Incident Energy (MeV)

101-Md-255

MAT 9956

Elastic
Legendre Coefficients

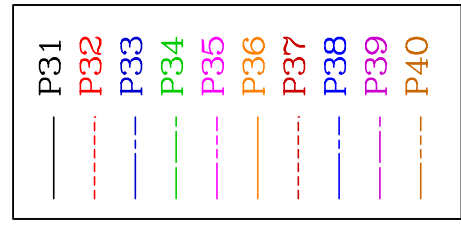
101-Md-255



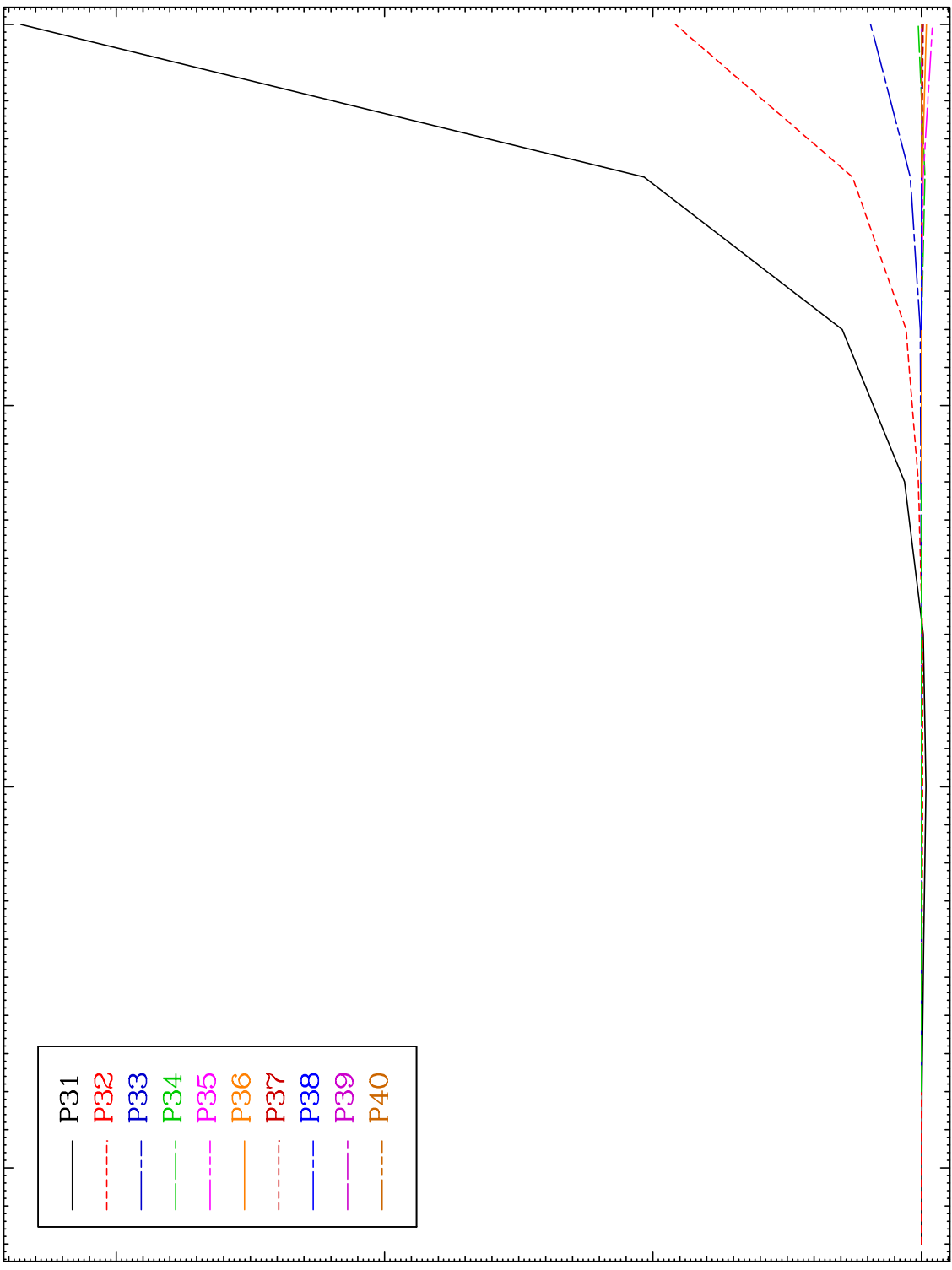
MAT 9956

Elastic Legendre Coefficients

101-Md-255



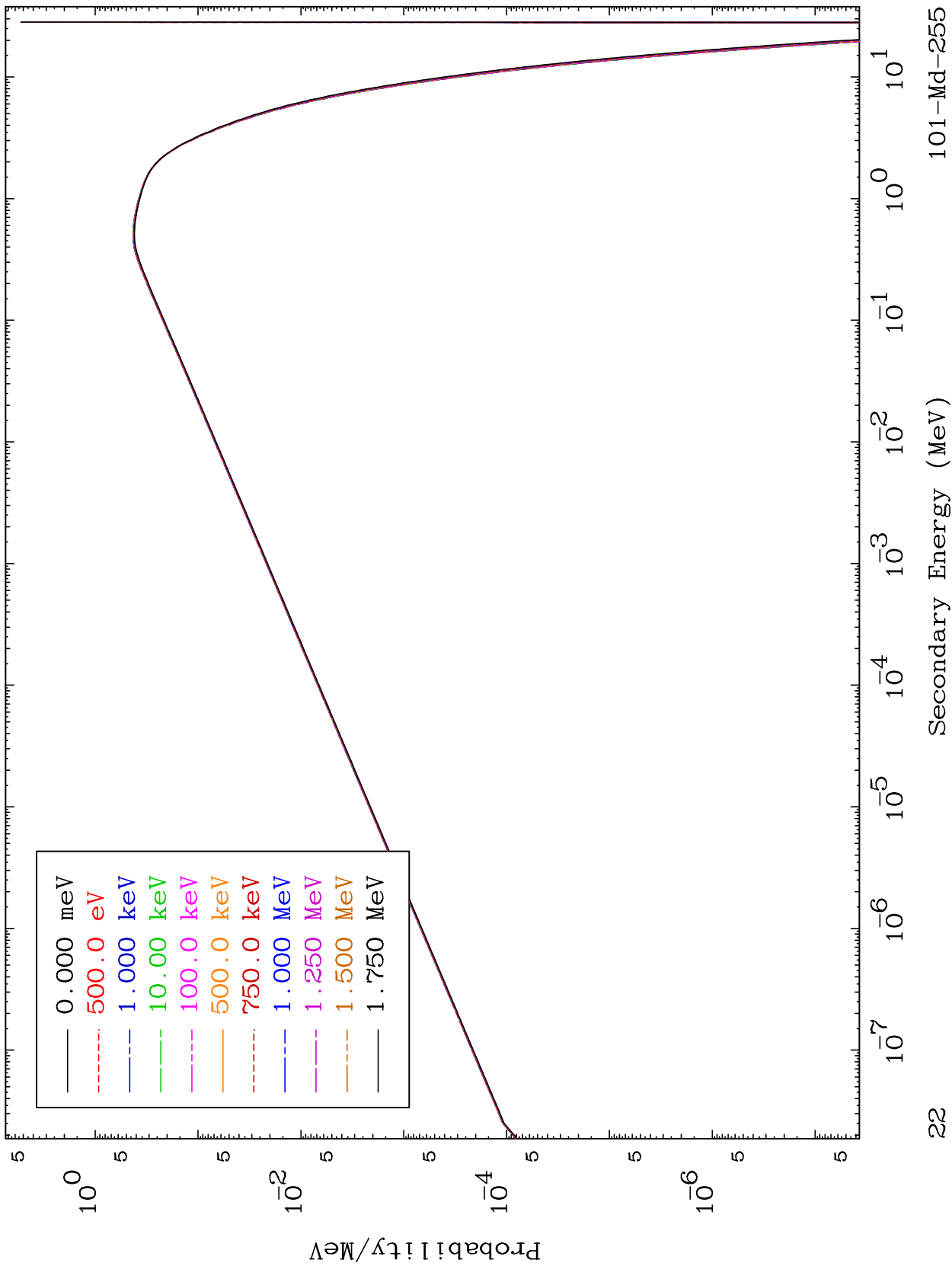
Legendre (CM)
 $\times 10^{-6}$



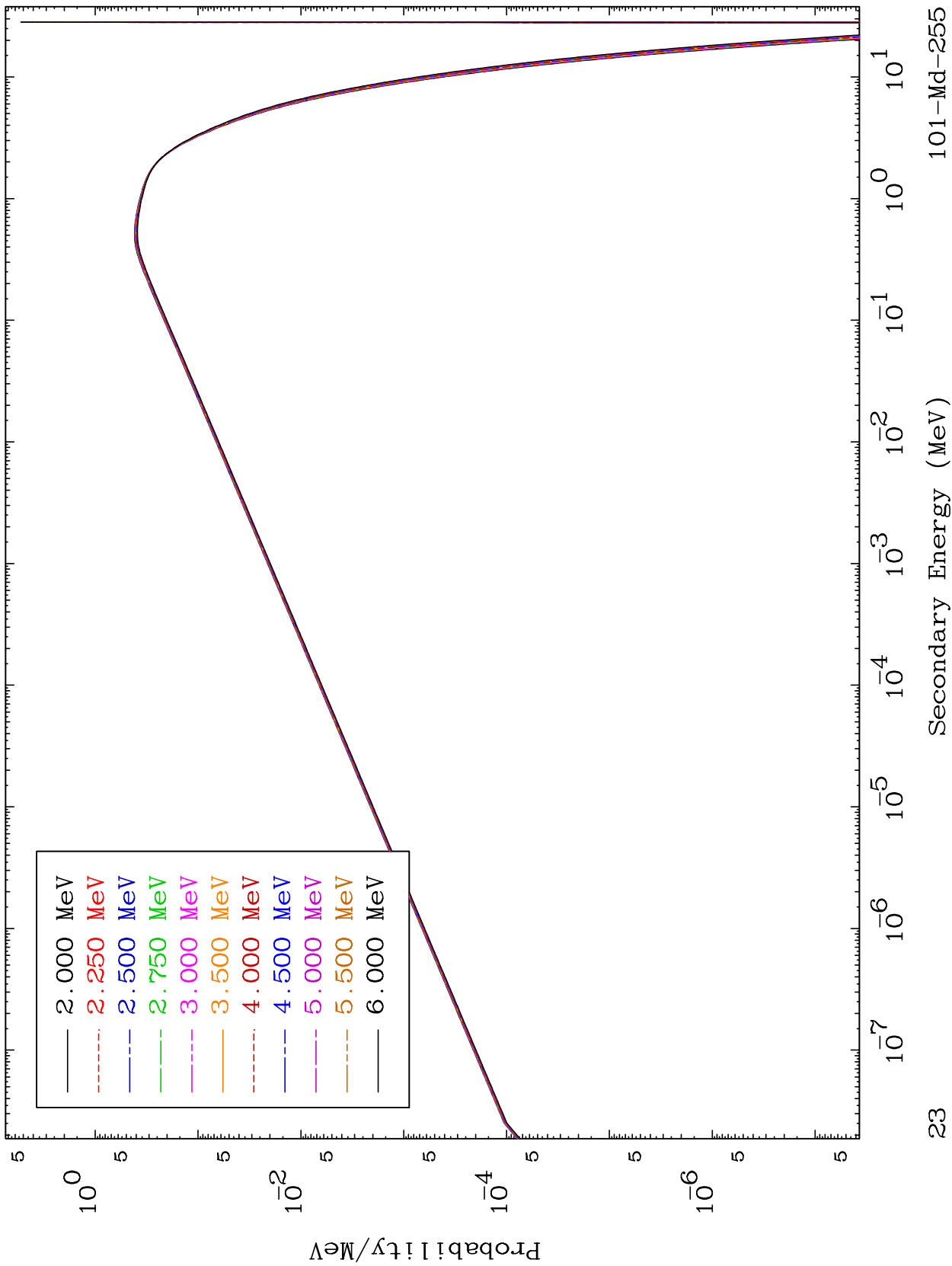
21

Incident Energy (MeV)

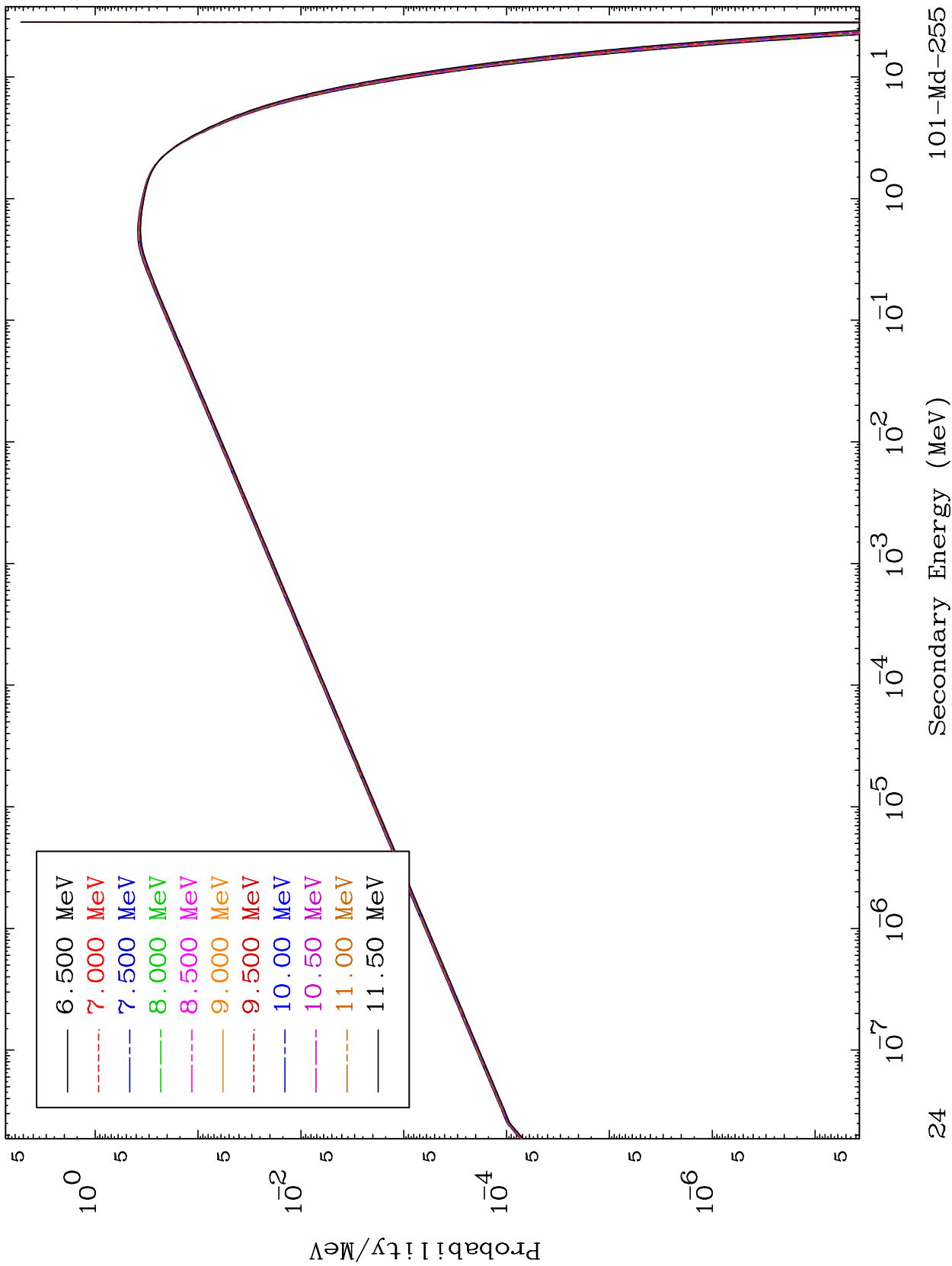
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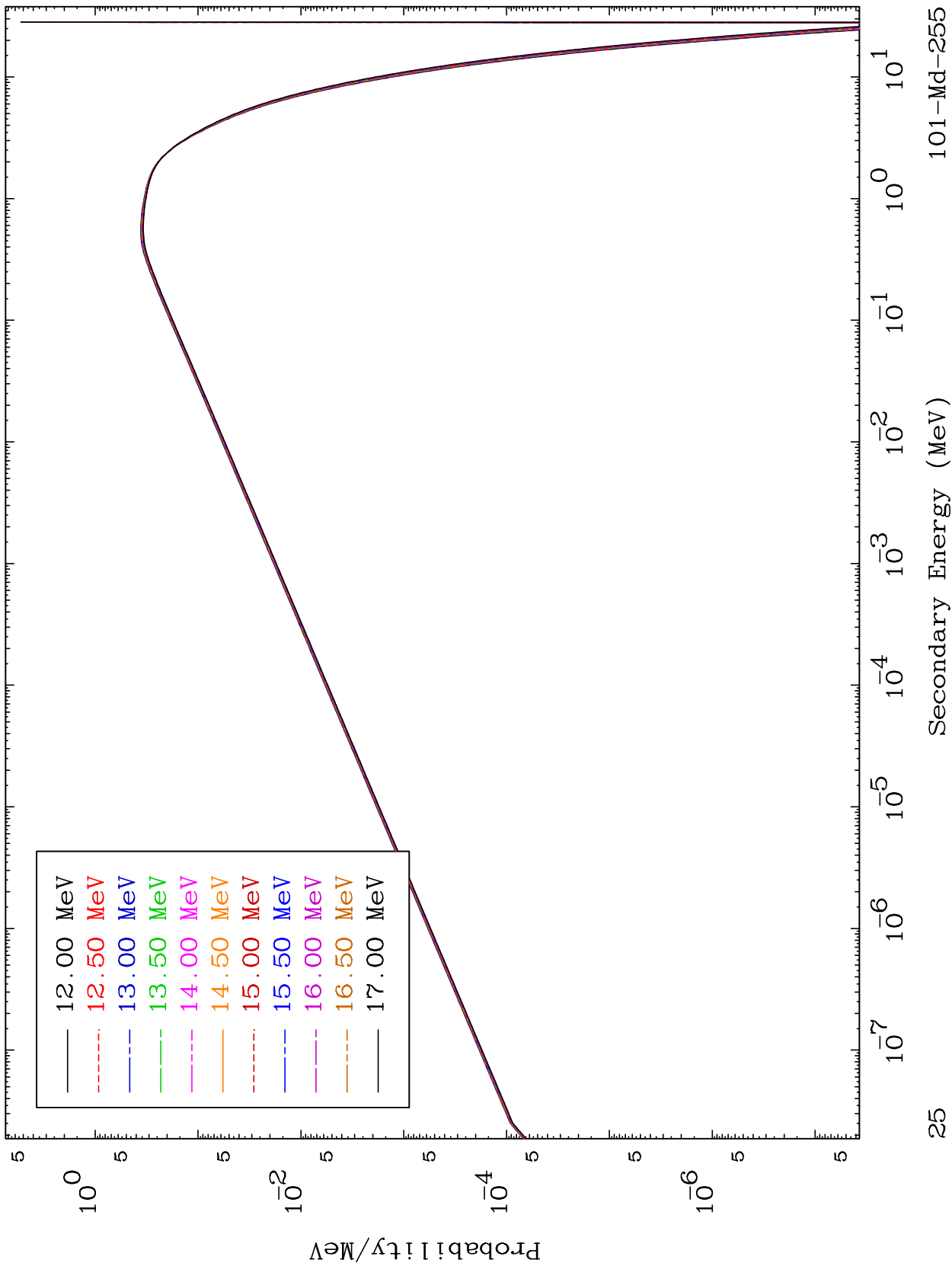


Fission
Energy Distribution

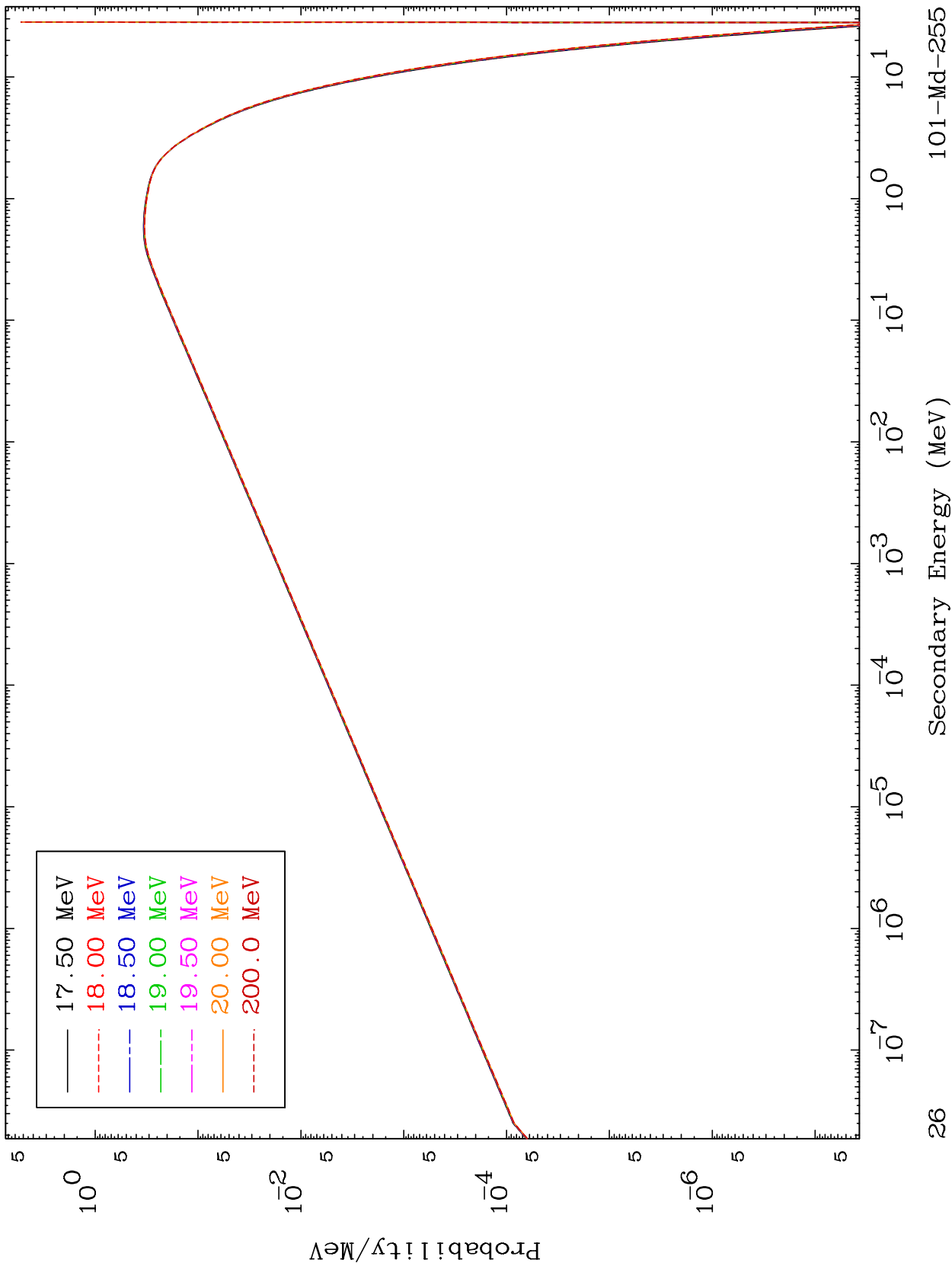


Fission
Energy Distribution





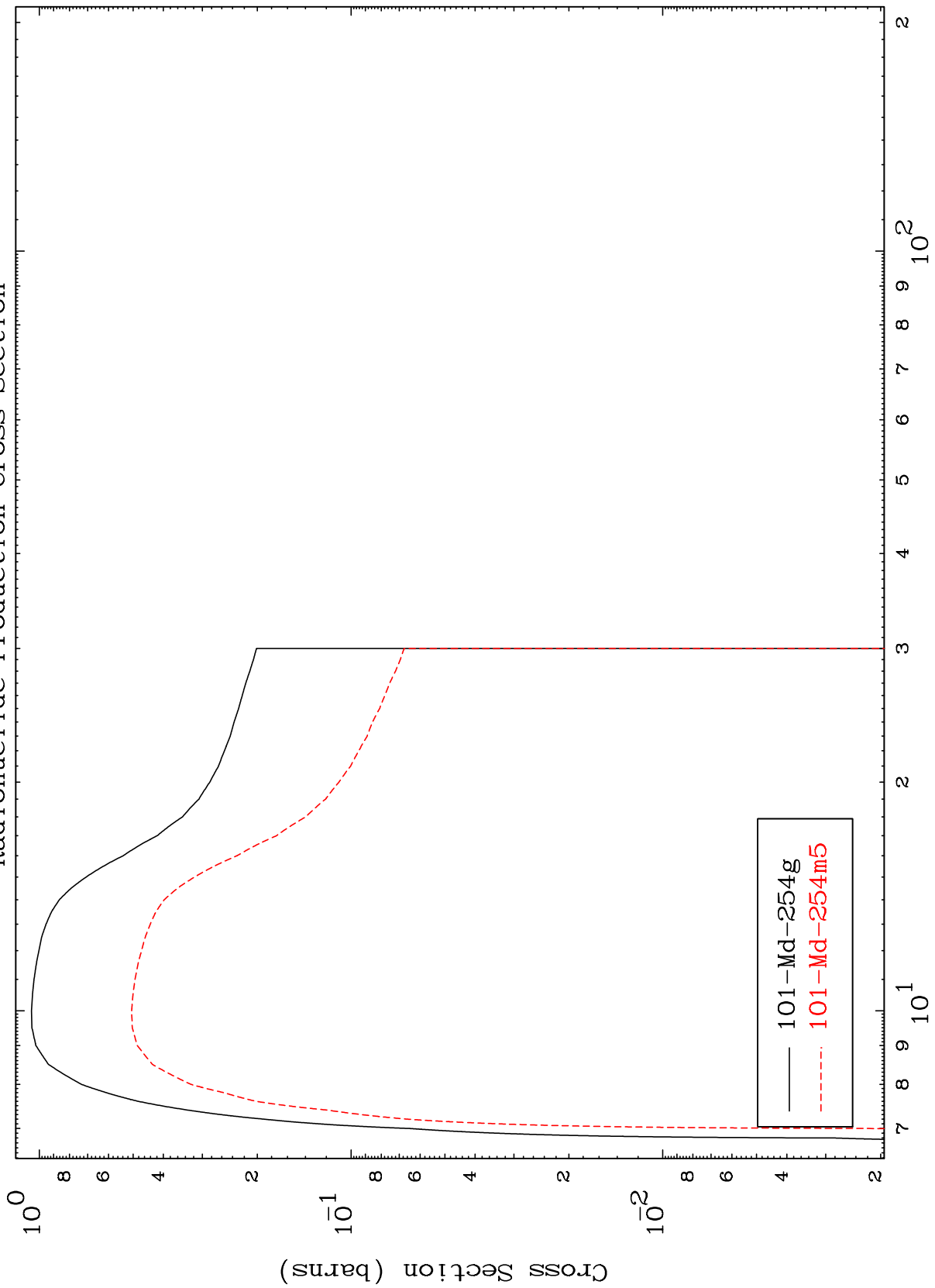
Fission
Energy Distribution



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101-Md-255

(n,2n)
Radionuclide Production Cross Section



— 101-Md-254g
- - - 101-Md-254m5

101-Md-255

Incident Energy (MeV)

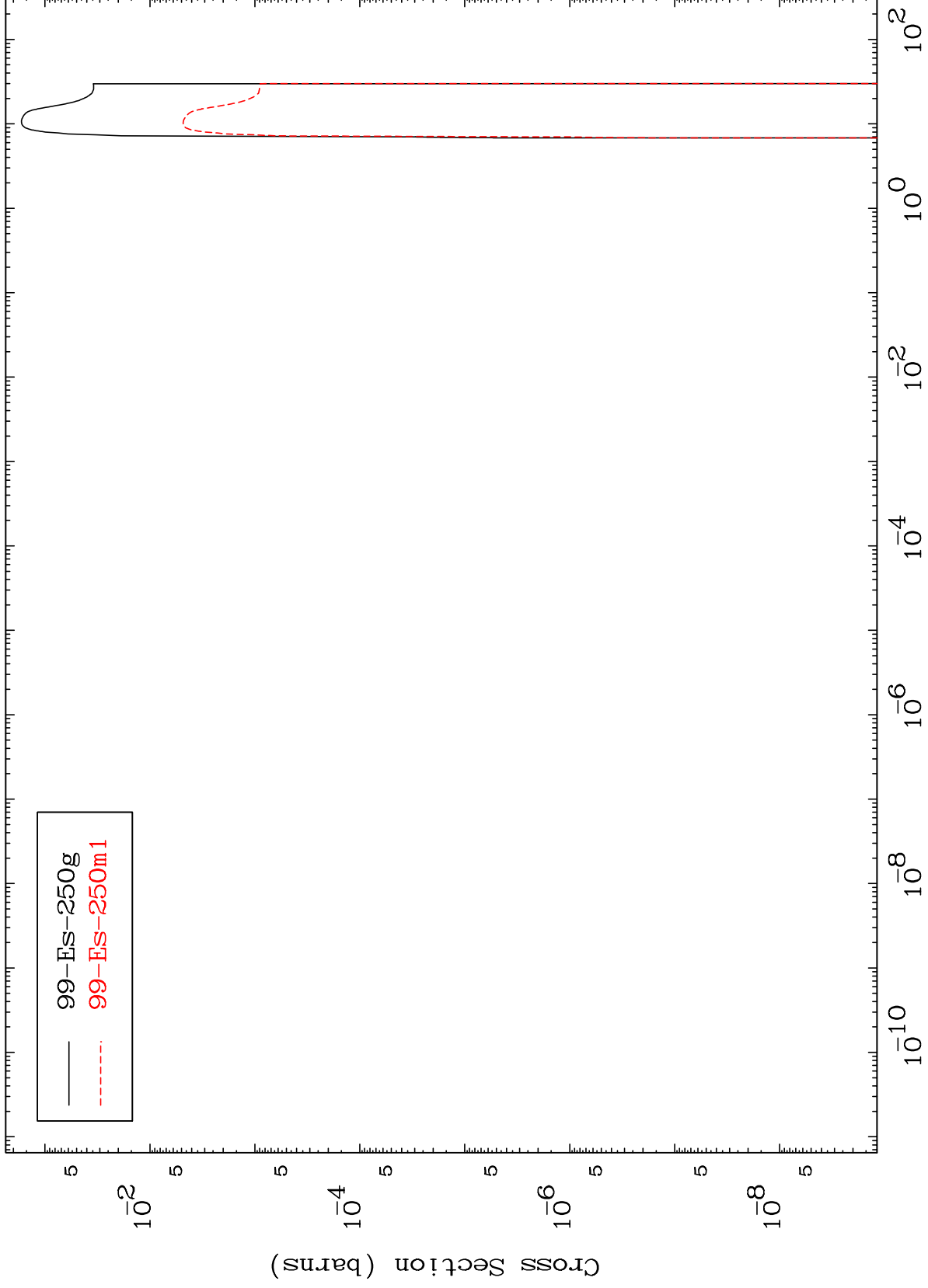
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$(n,2n) \alpha$

101-Md-255

Radionuclide Production Cross Section



MAT 9956

101-Md-255

Radionuclide Production Cross Section
(n,2α)

