

Program EVALPLOT
(Version 2015-2)

by

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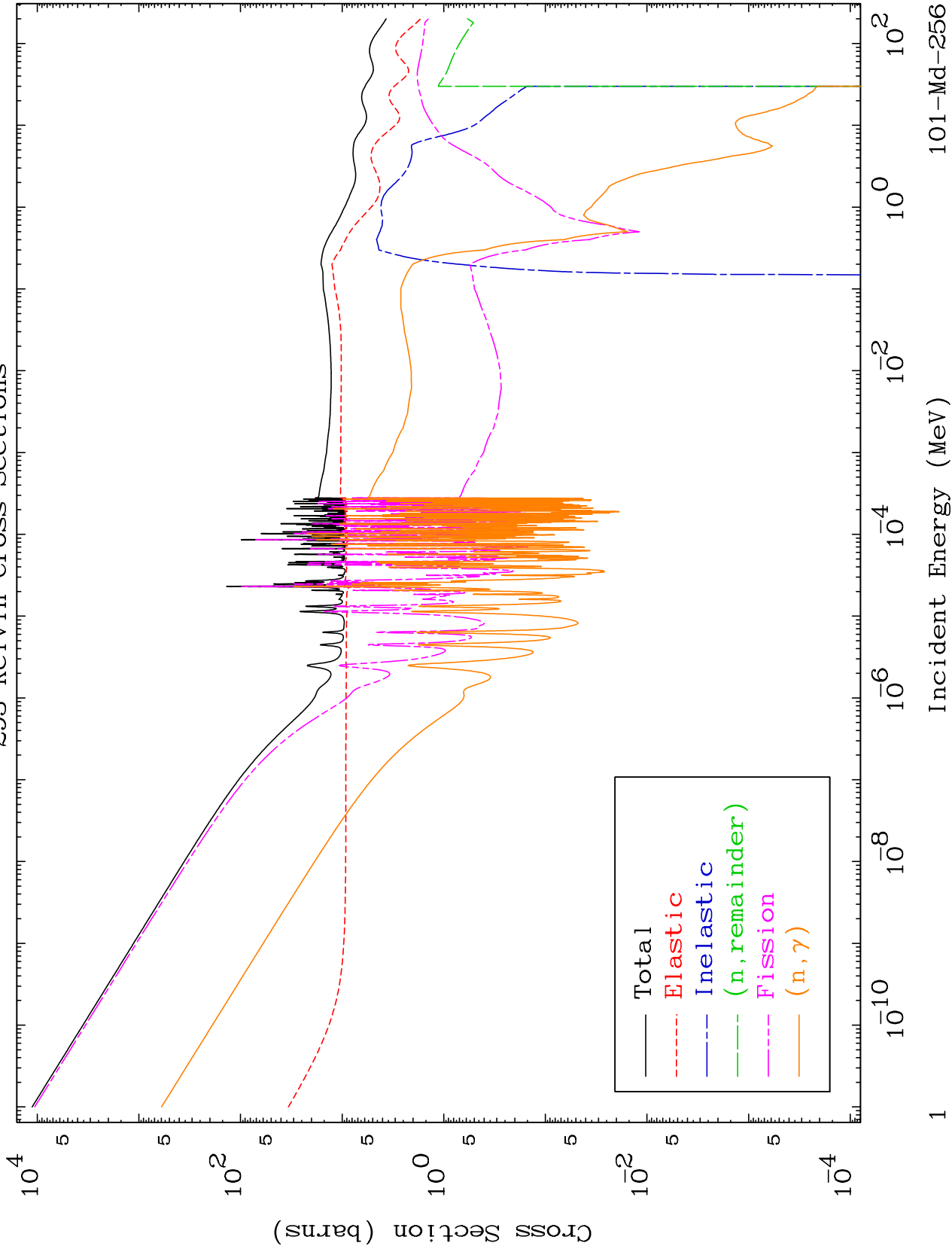
Press Mouse Button to Start

MAT 9957

Major

293 Kelvin Cross Sections

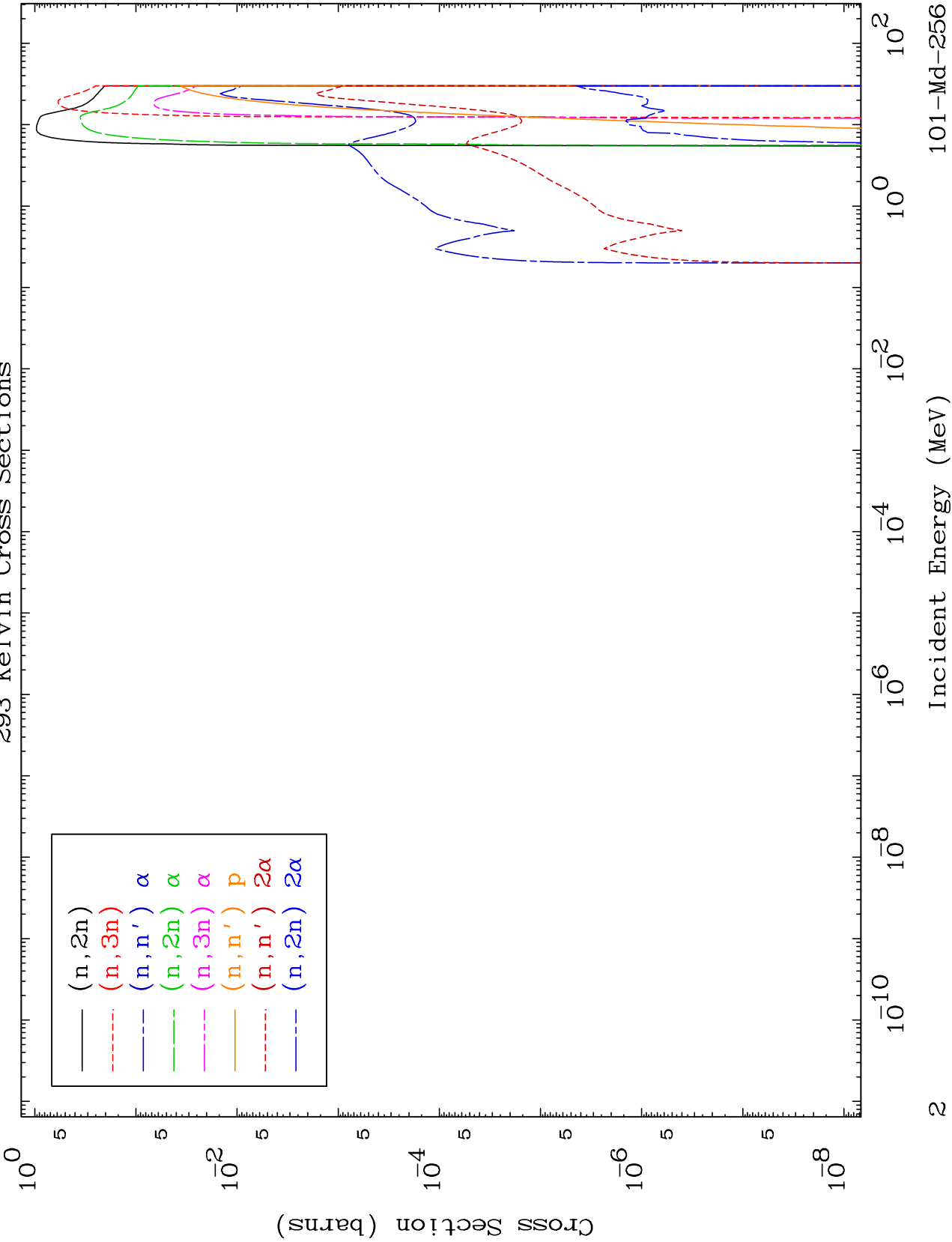
101-Md-256



MAT 9957

Neutron Production
293 Kelvin Cross Sections

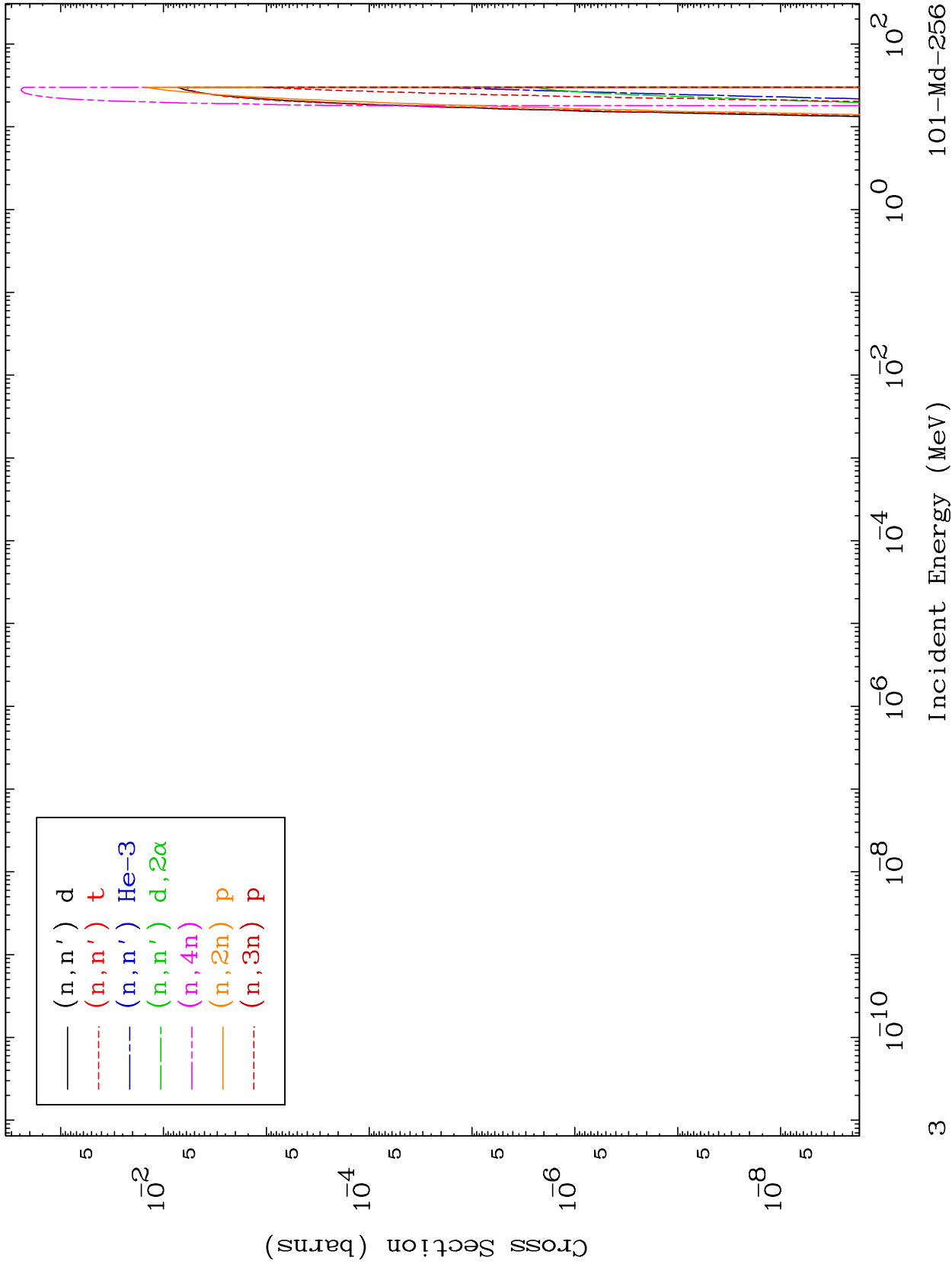
101-Md-256



MAT 9957

Neutron Production
293 Kelvin Cross Sections

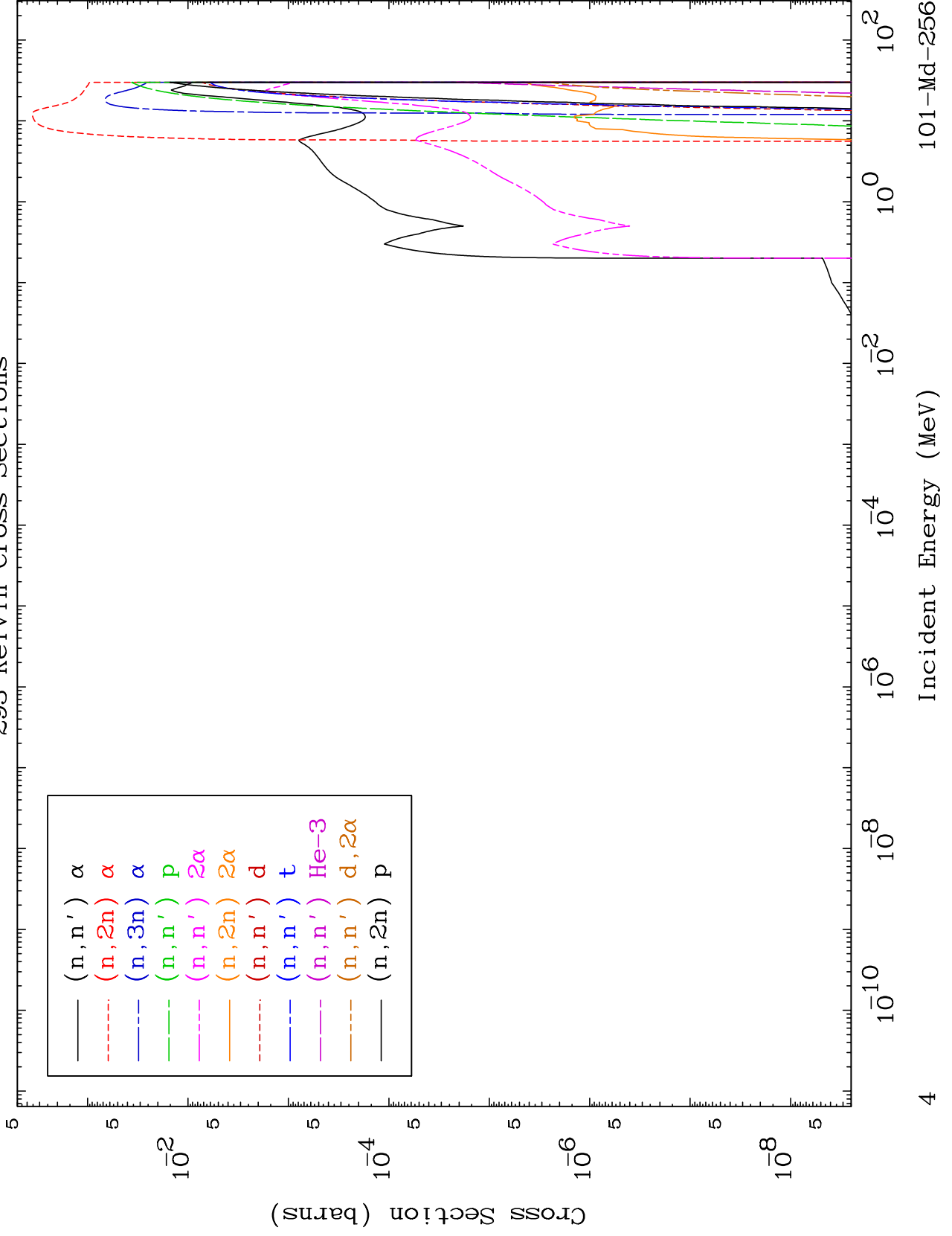
101-Md-256



MAT 9957

Charged Particle
293 Kelvin Cross Sections

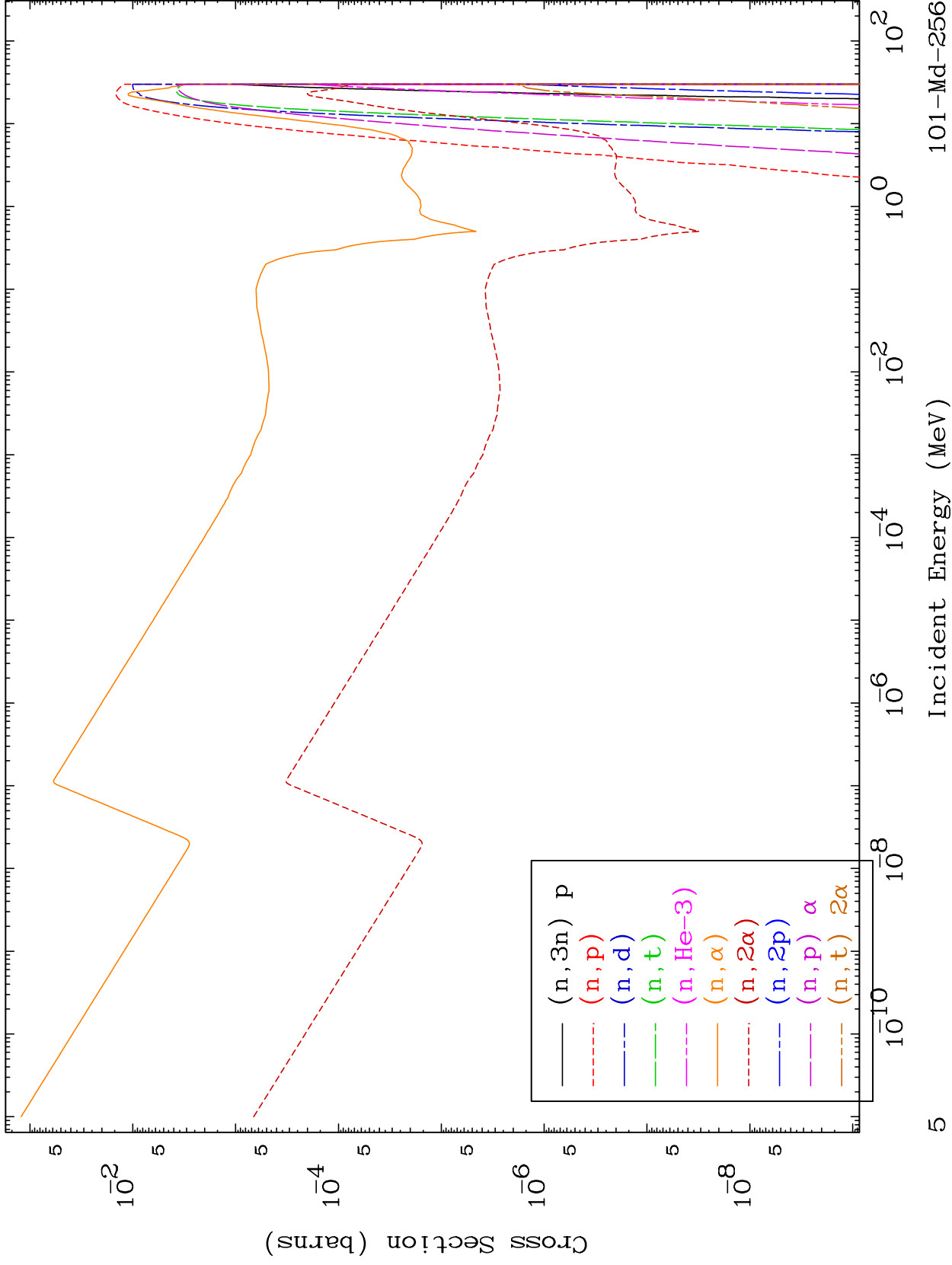
101-Md-256



MAT 9957

Charged Particle
293 Kelvin Cross Sections

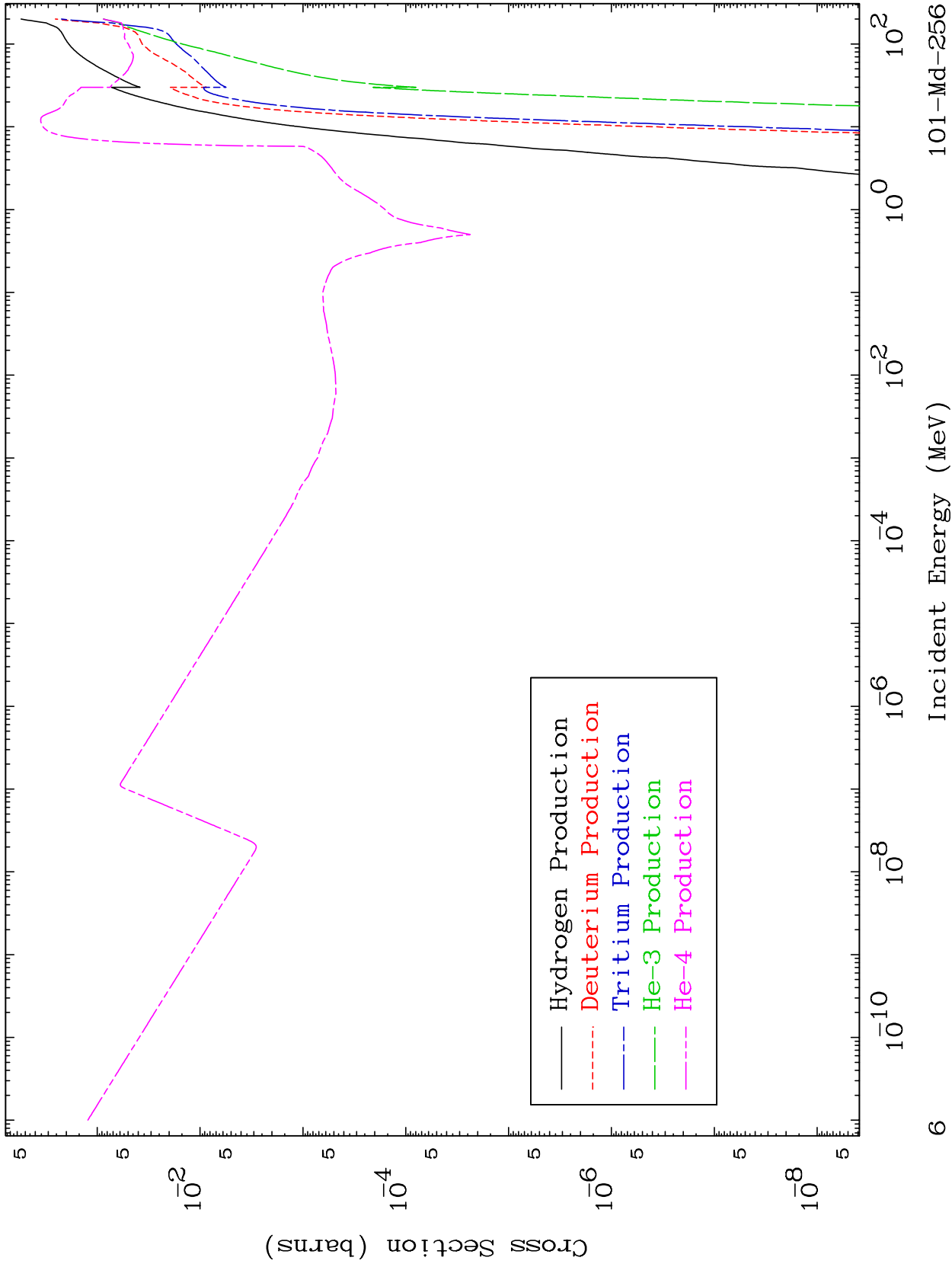
101-Md-256



MAT 9957

Particle Production
293 Kelvin Cross Sections

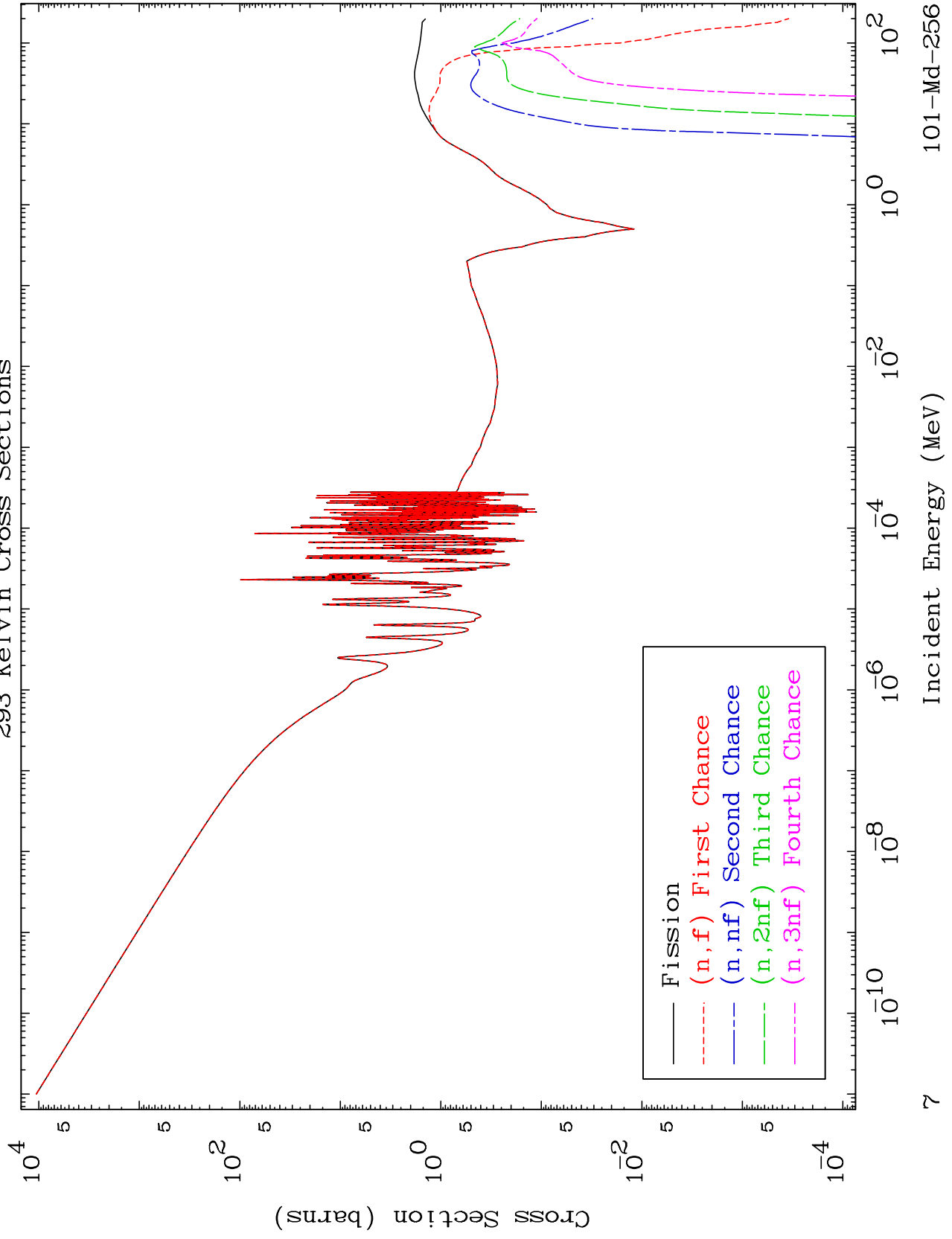
101-Md-256



MAT 9957

Fission
293 Kelvin Cross Sections

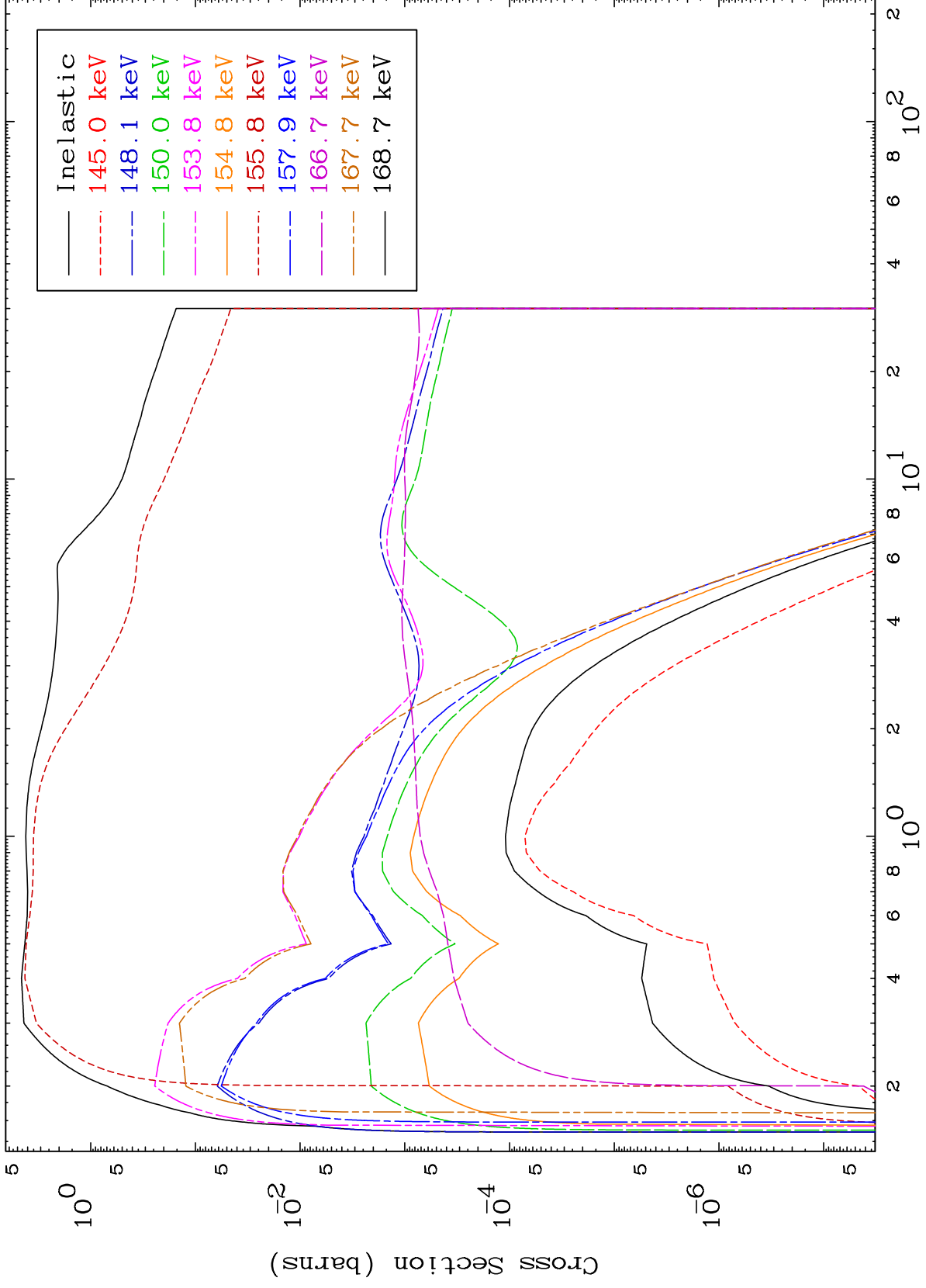
101-Md-256



MAT 9957

(n,n') Level
293 Kelvin Cross Sections

101-Md-256



8

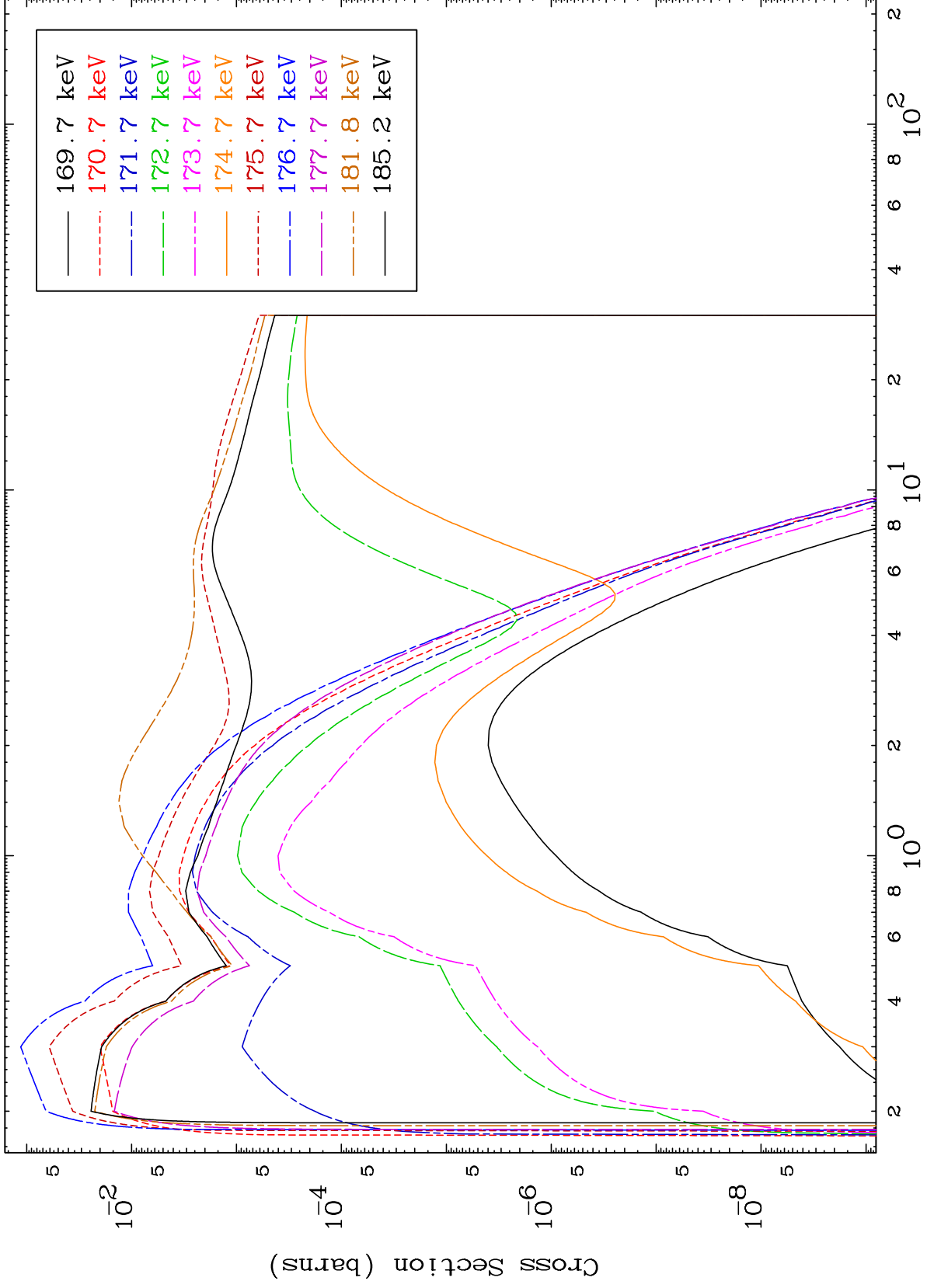
Incident Energy (MeV)

101-Md-256

MAT 9957

(n,n') Level
293 Kelvin Cross Sections

101-Md-256



9

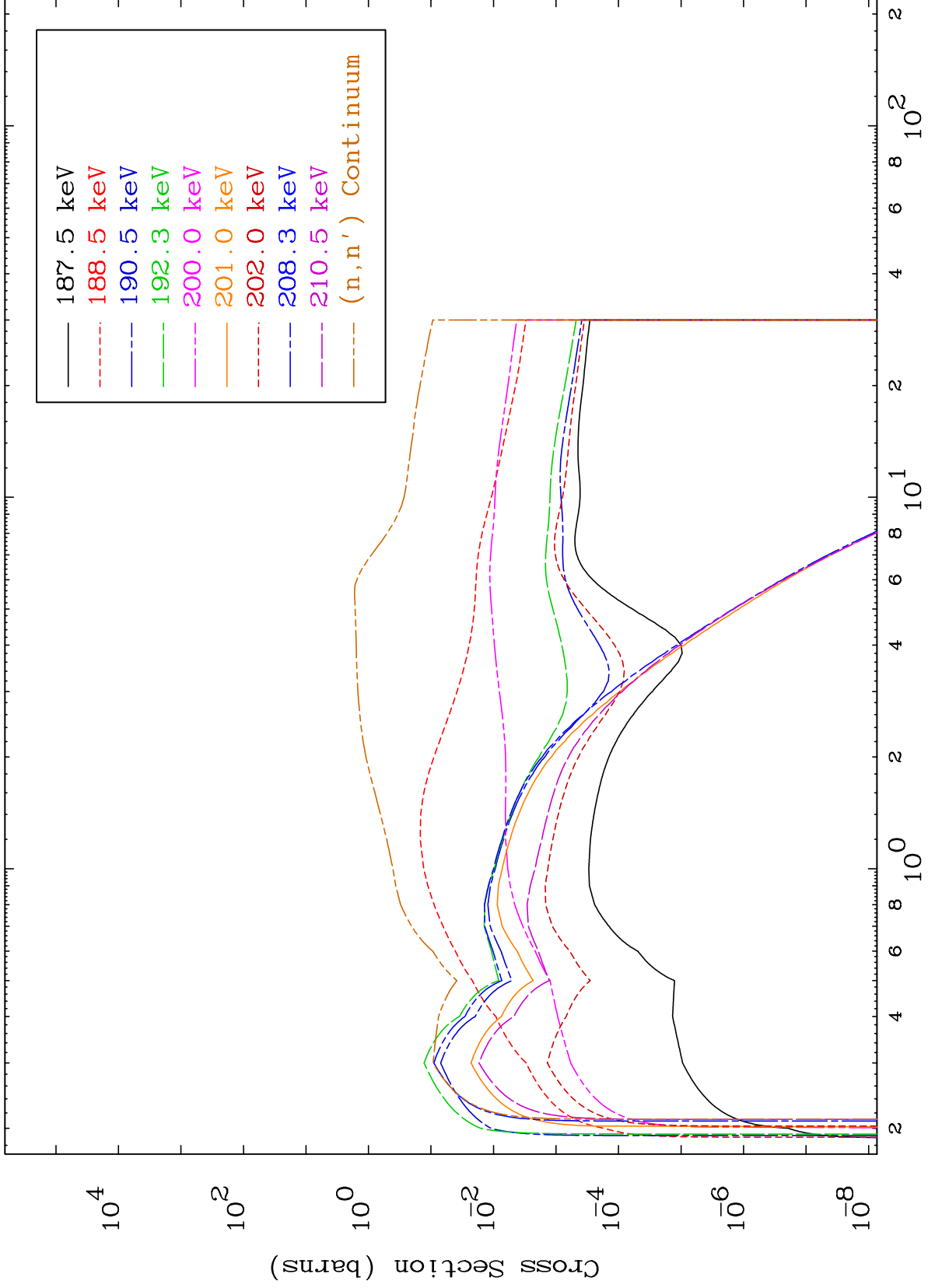
Incident Energy (MeV)

101-Md-256

MAT 9957

(n,n') Level
293 Kelvin Cross Sections

101-Md-256



10

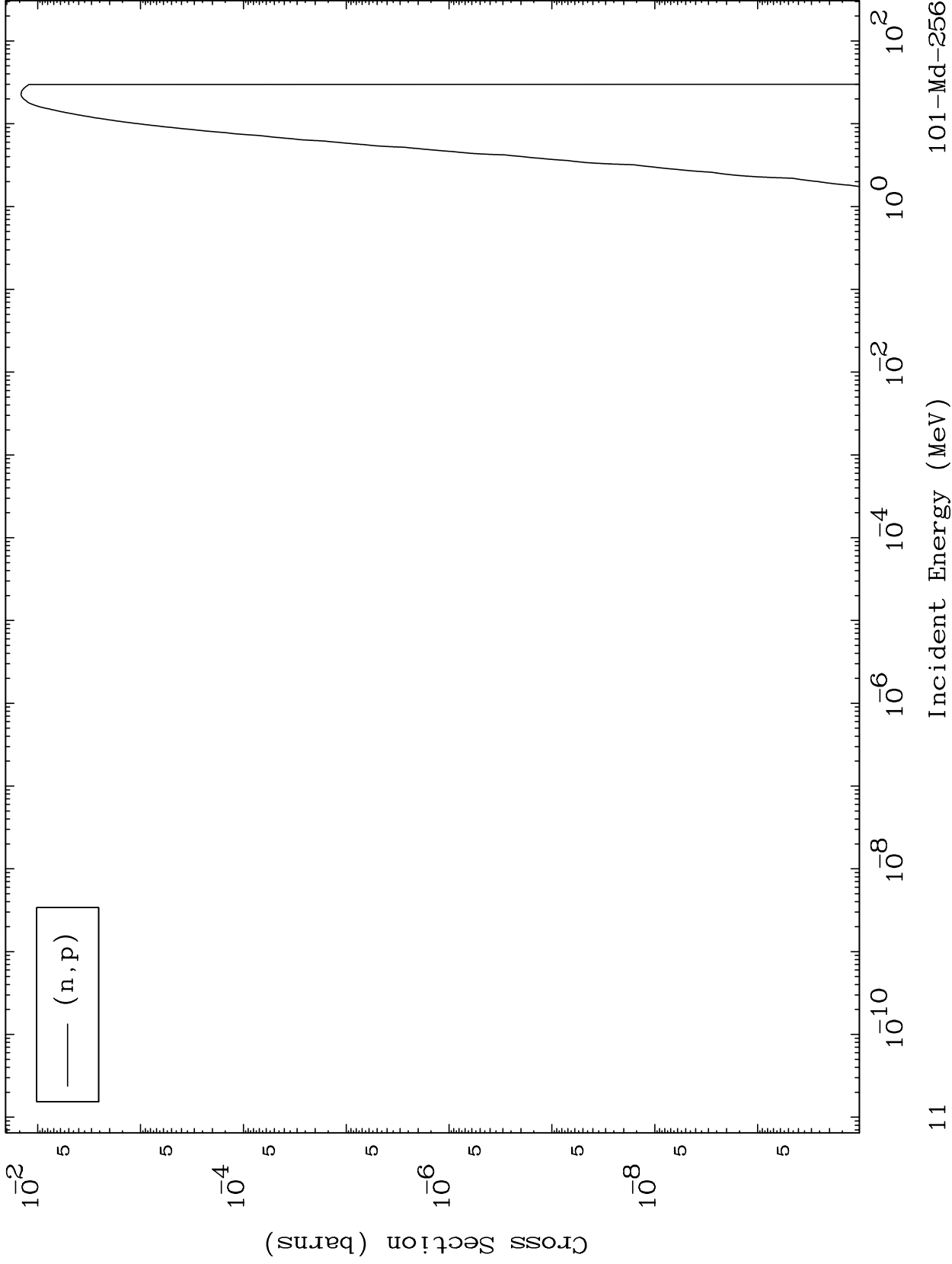
Incident Energy (MeV)

101-Md-256

MAT 9957

(n,p) Levels
293 Kelvin Cross Sections

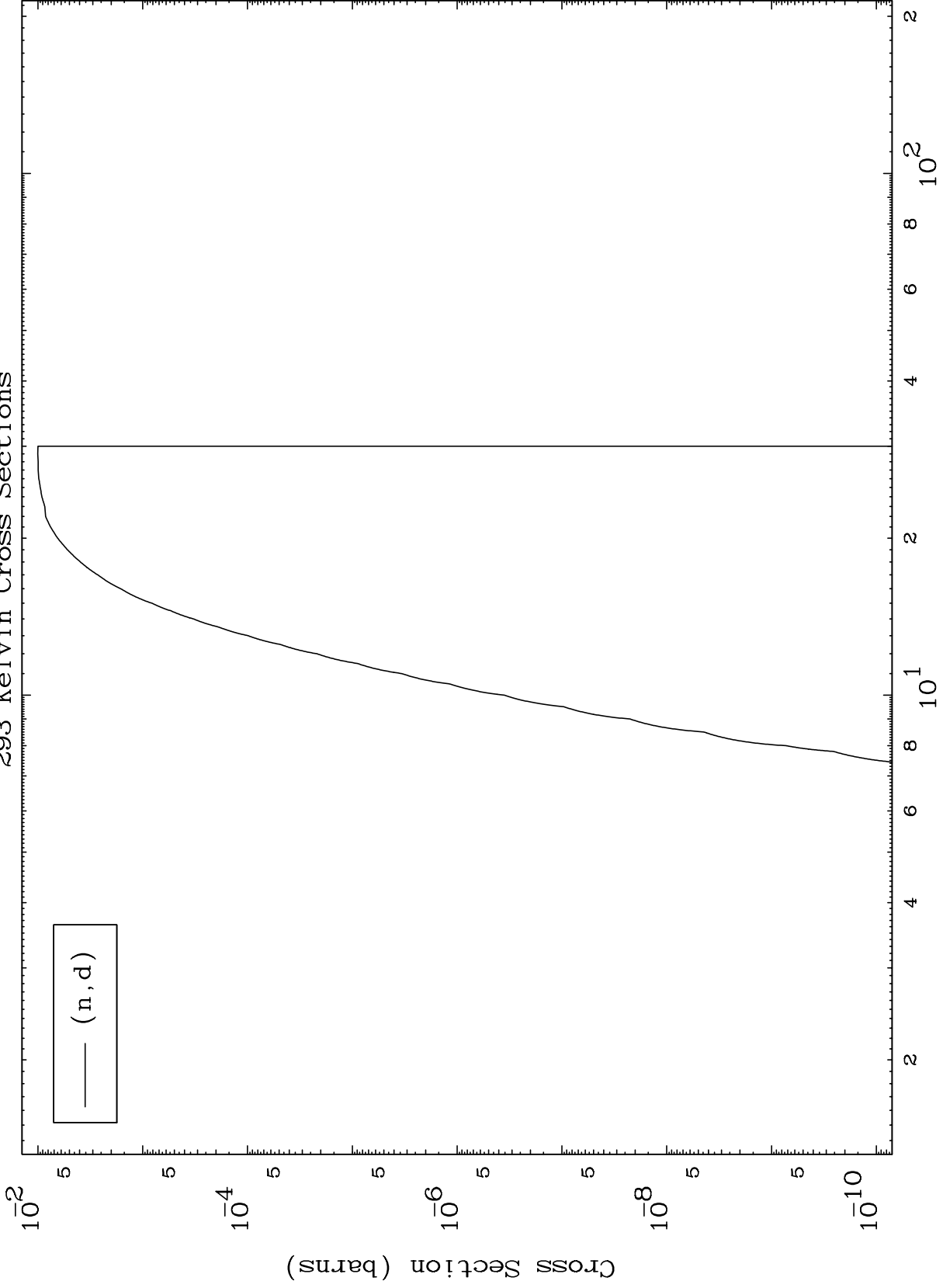
101-Md-256



MAT 9957

(n,d) Levels
293 Kelvin Cross Sections

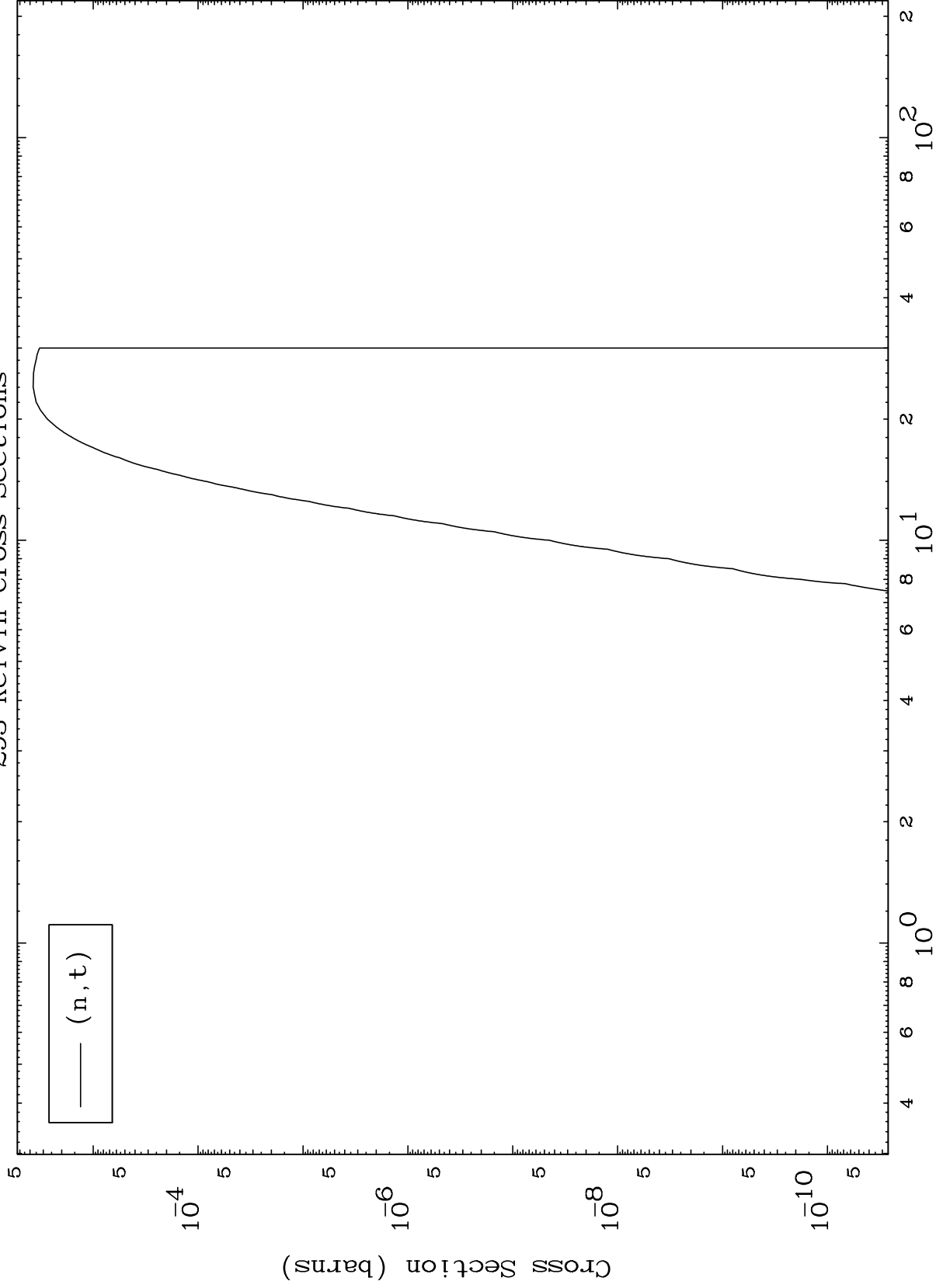
101-Md-256



MAT 9957

(n,t) Levels
293 Kelvin Cross Sections

101-Md-256



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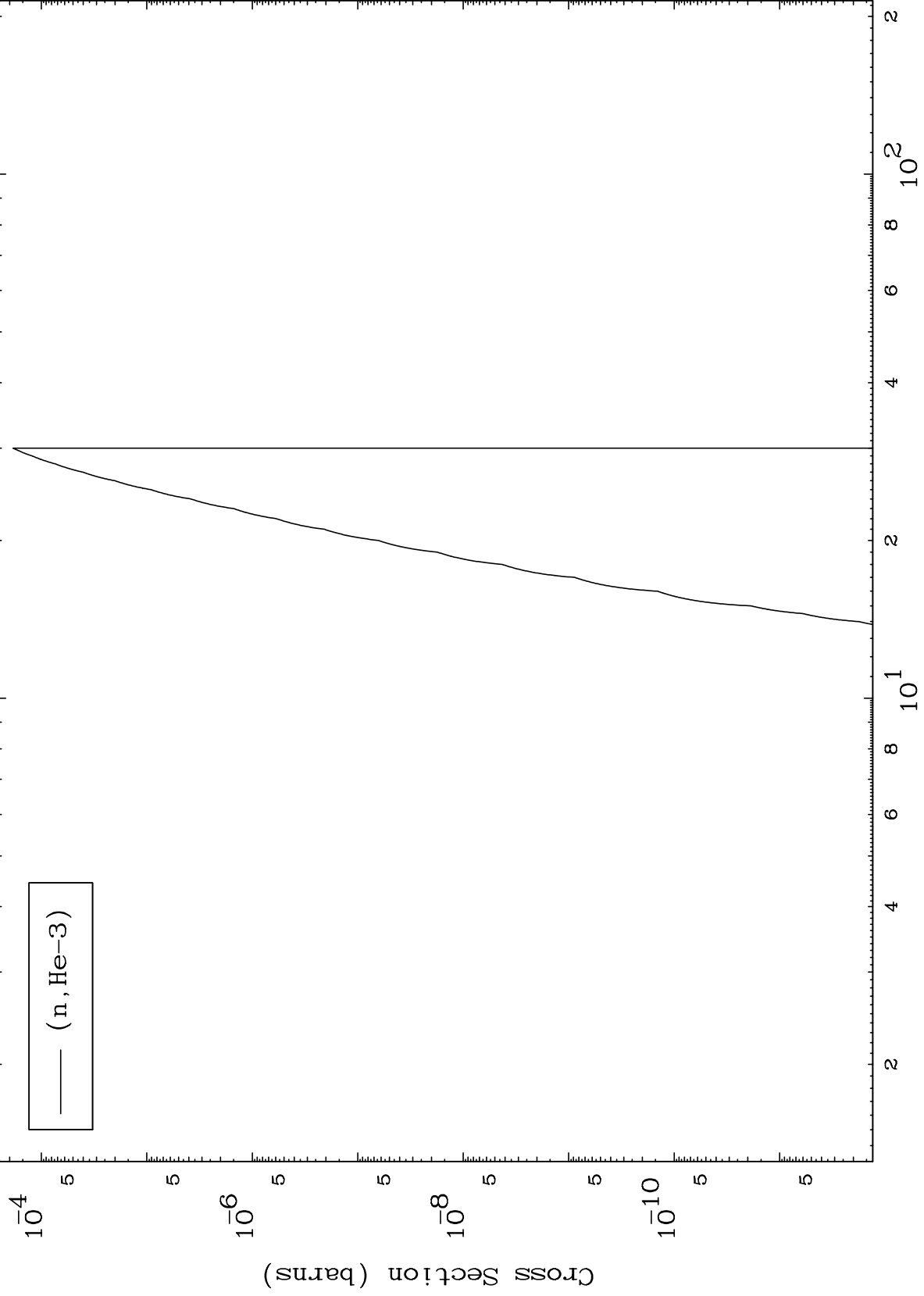
Incident Energy (MeV)

101-Md-256

MAT 9957

(n,He3) Levels
293 Kelvin Cross Sections

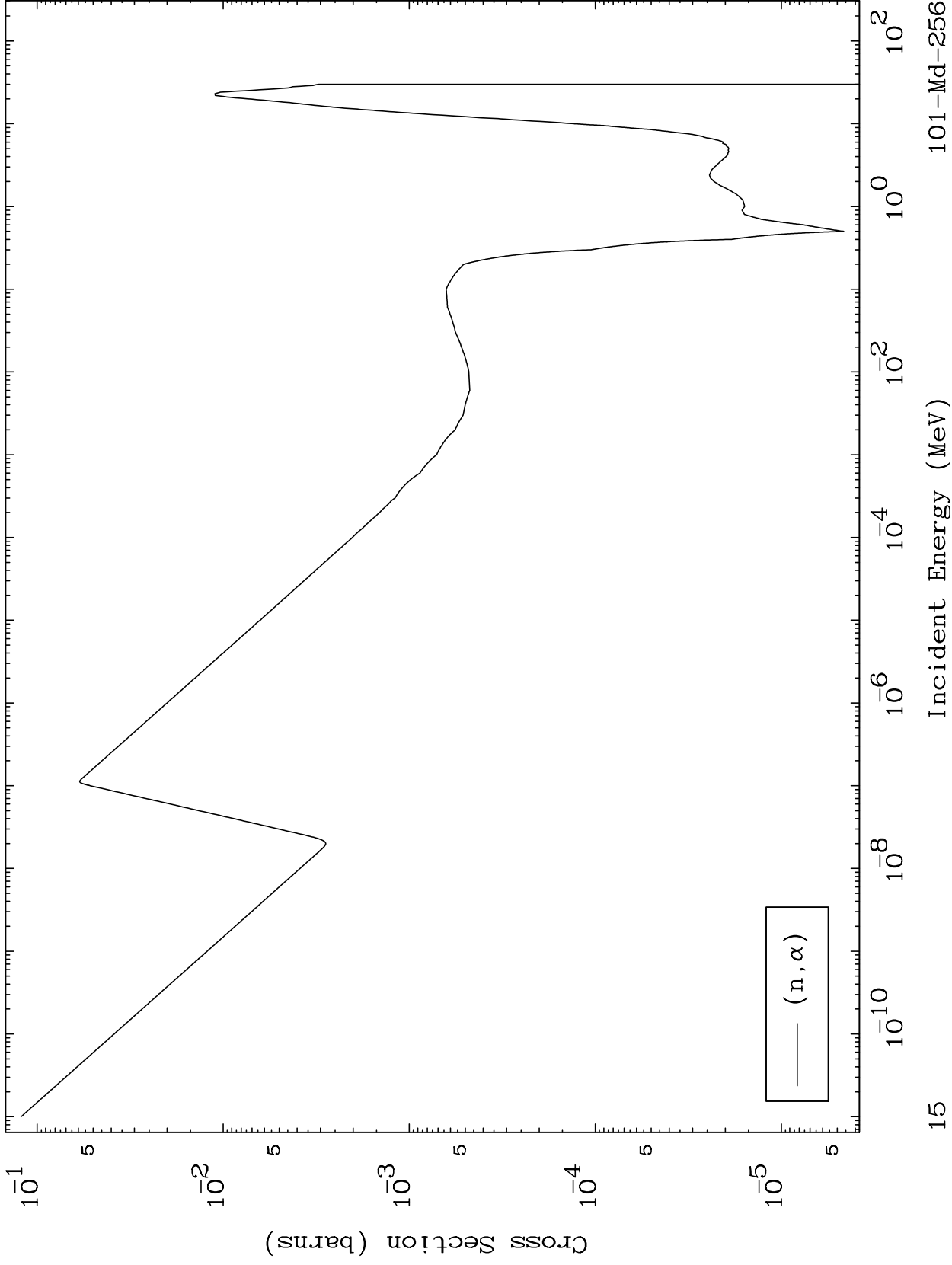
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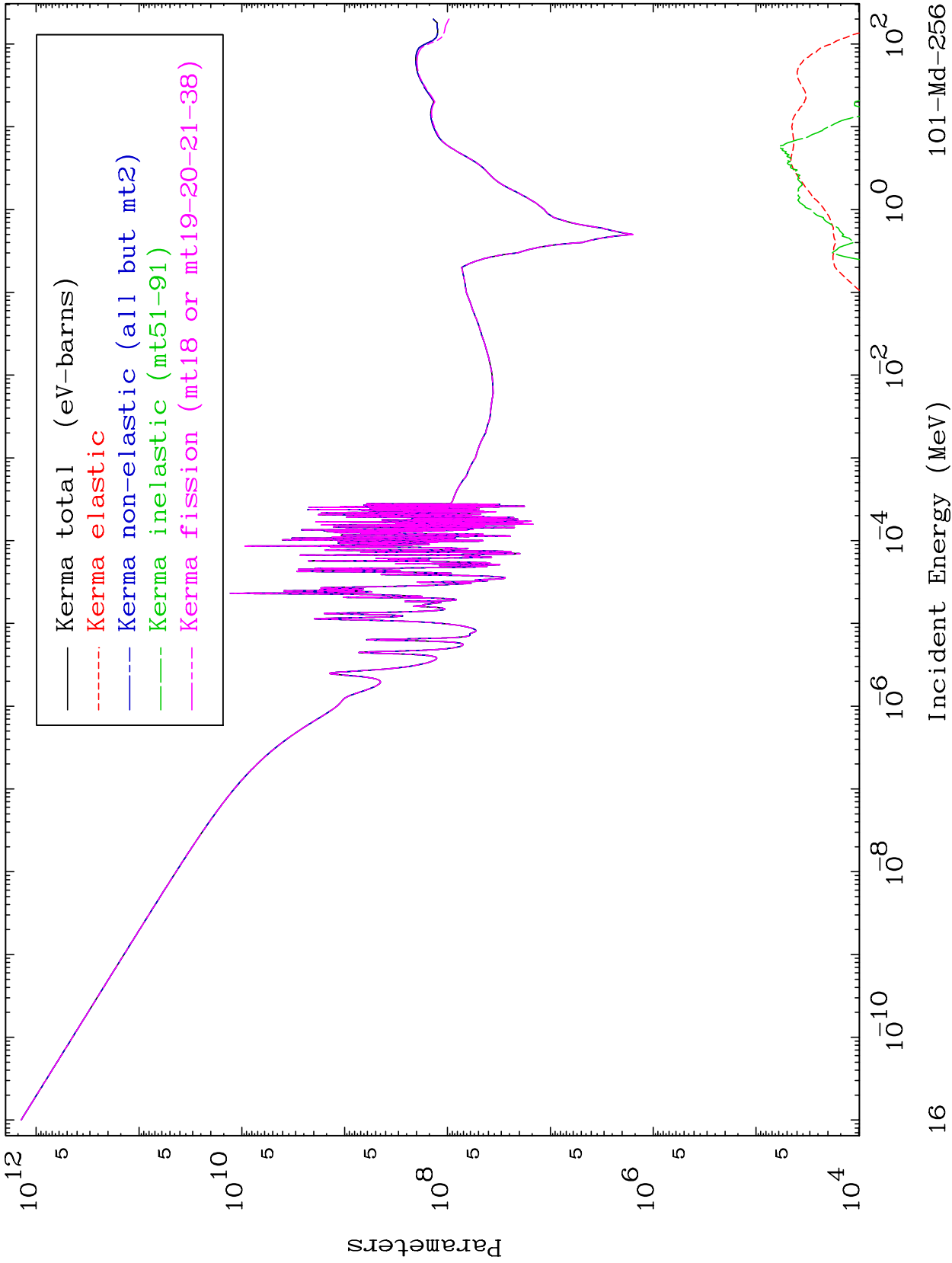
MAT 9957

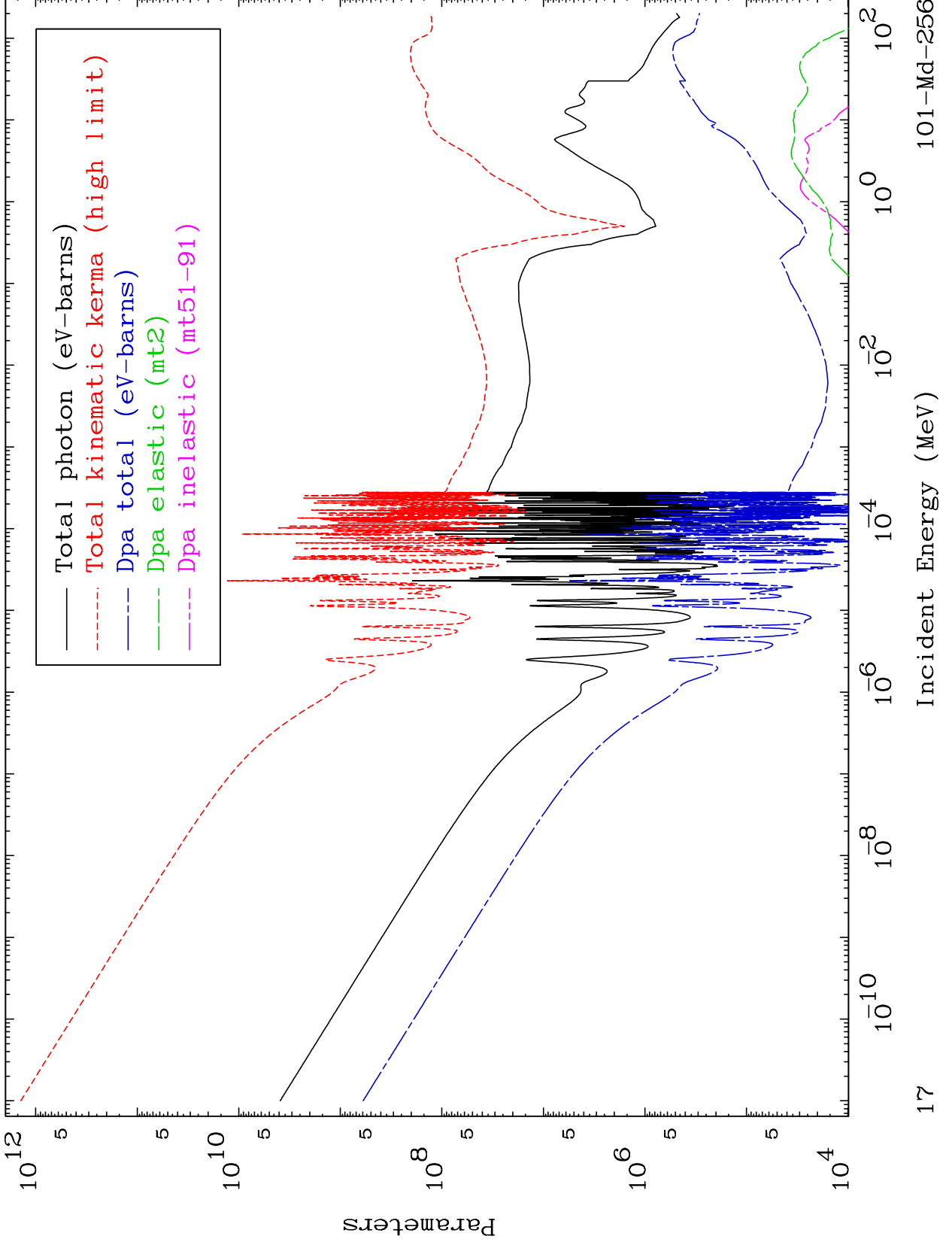
(n,α) Levels
293 Kelvin Cross Sections

101-Md-256



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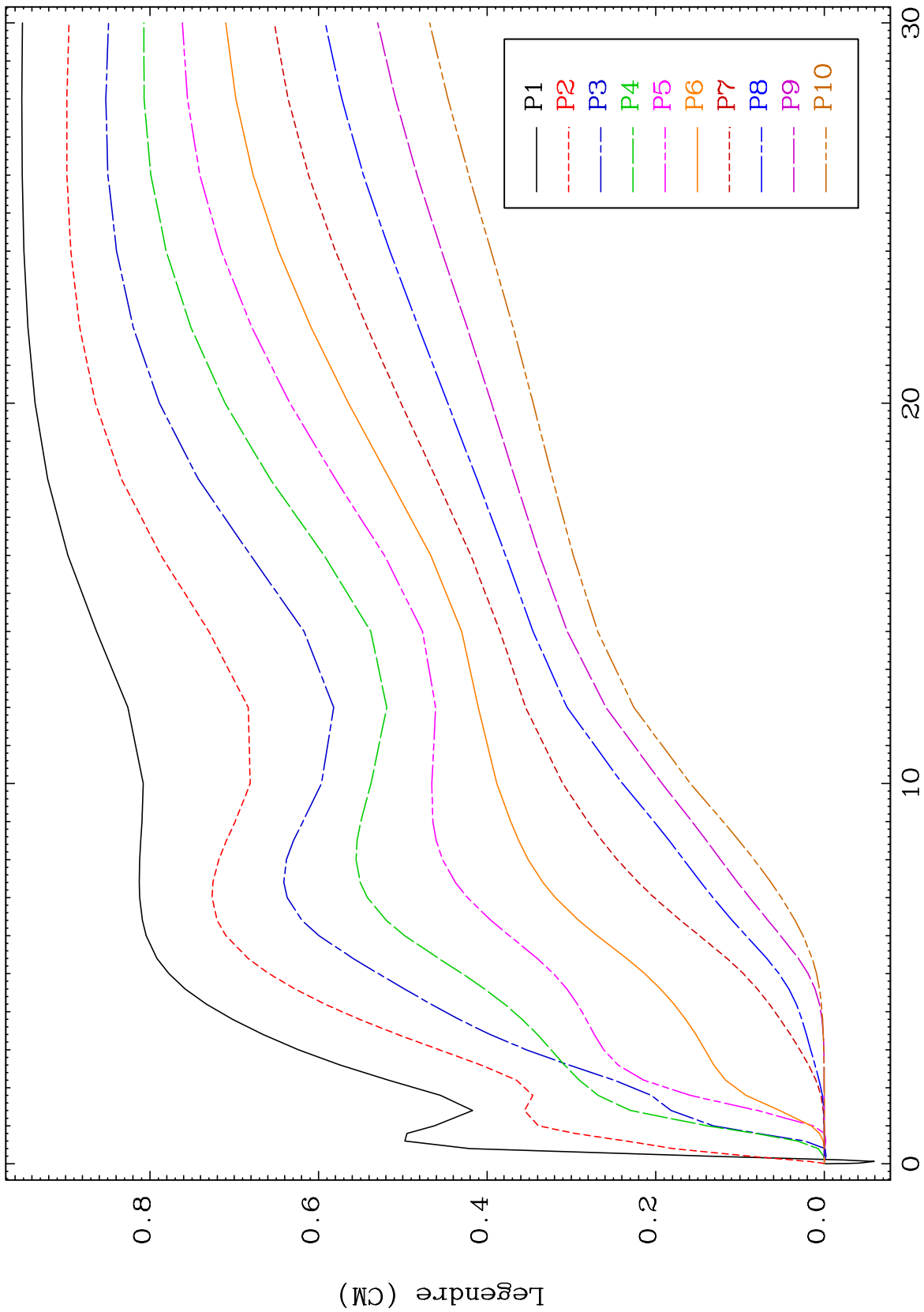




MAT 9957

Elastic Legendre Coefficients

101-Md-256



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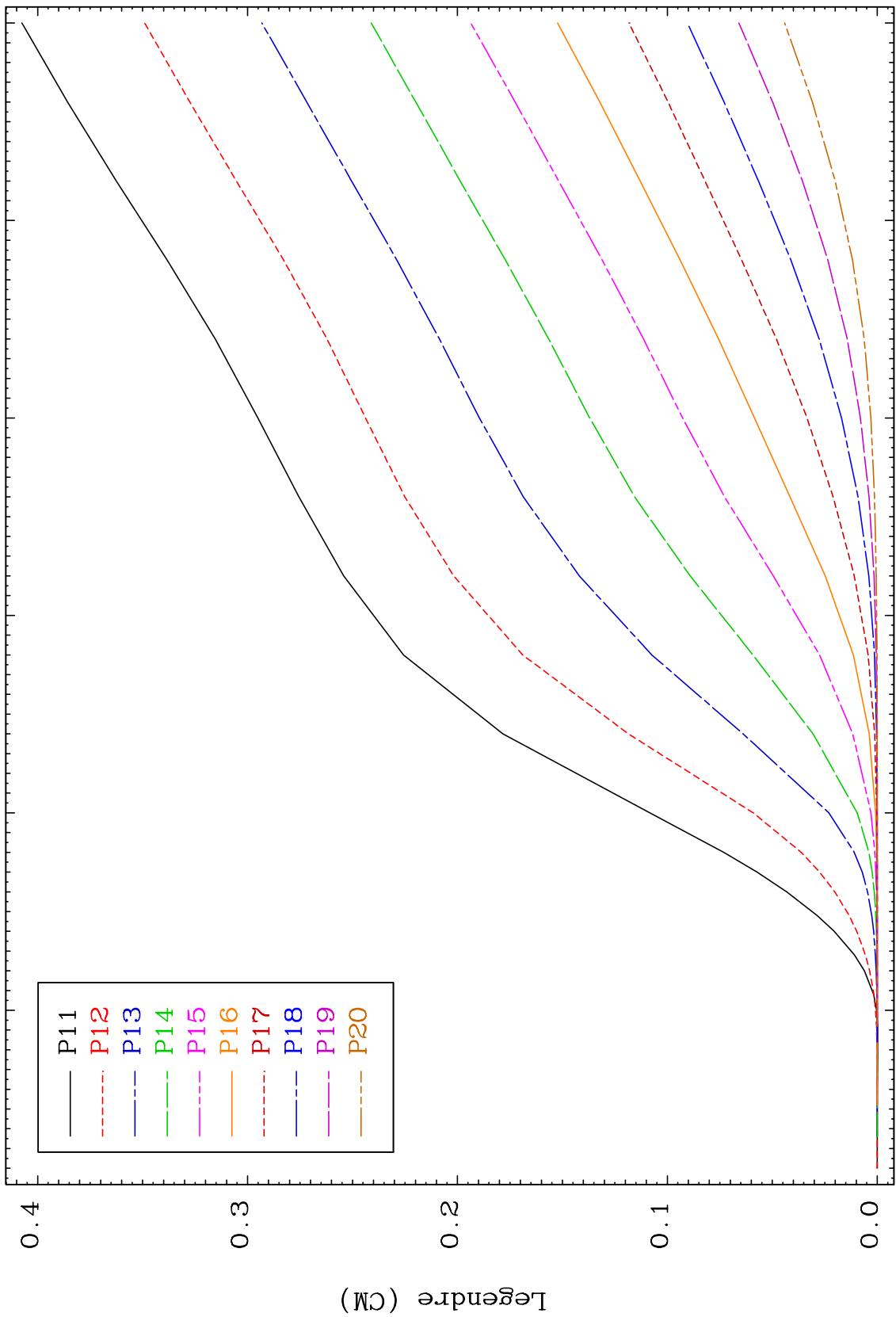
Incident Energy (MeV)

101-Md-256

MAT 9957

Elastic Legendre Coefficients

101-Md-256



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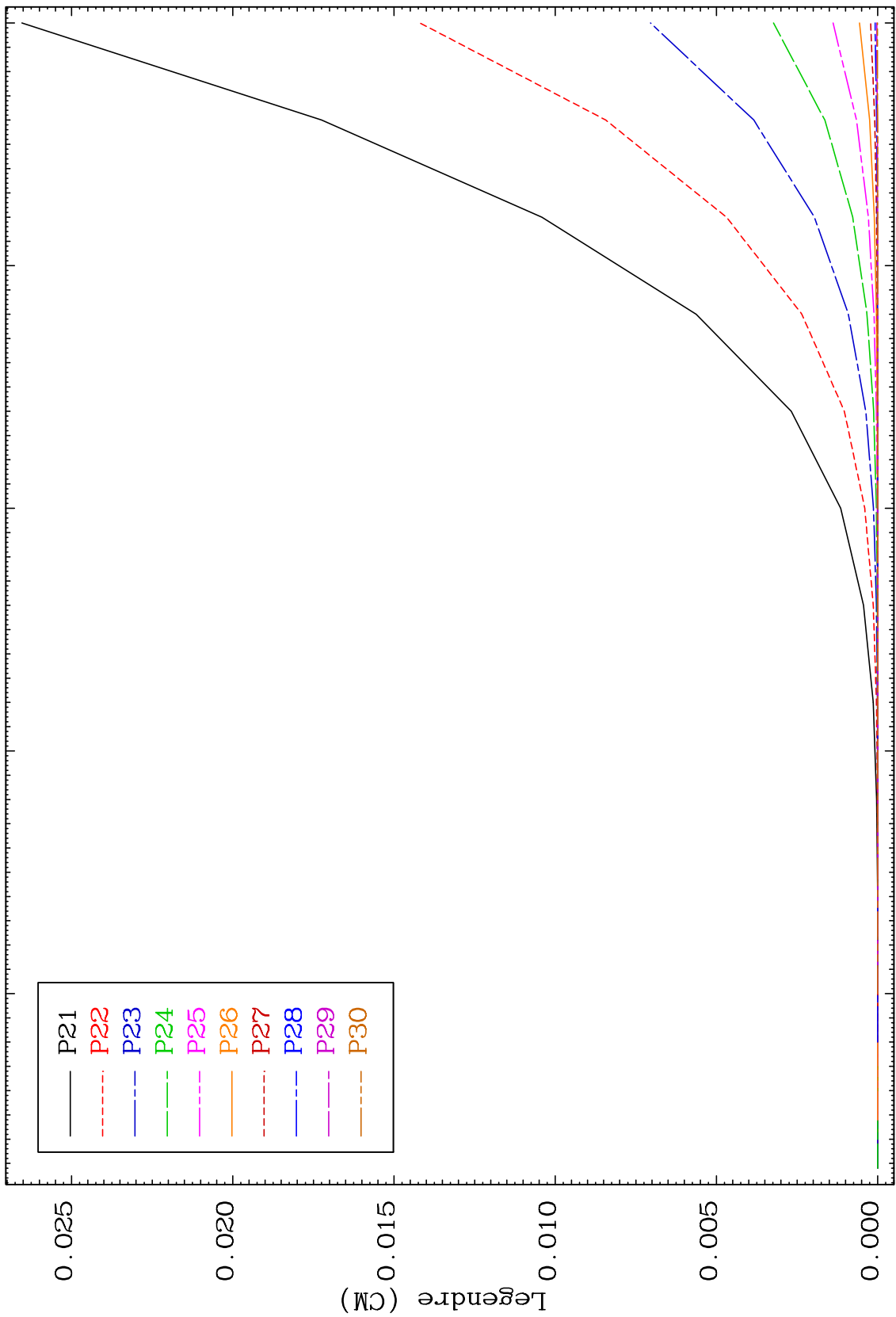
Incident Energy (MeV)

101-Md-256

MAT 9957

Elastic Legendre Coefficients

101-Md-256



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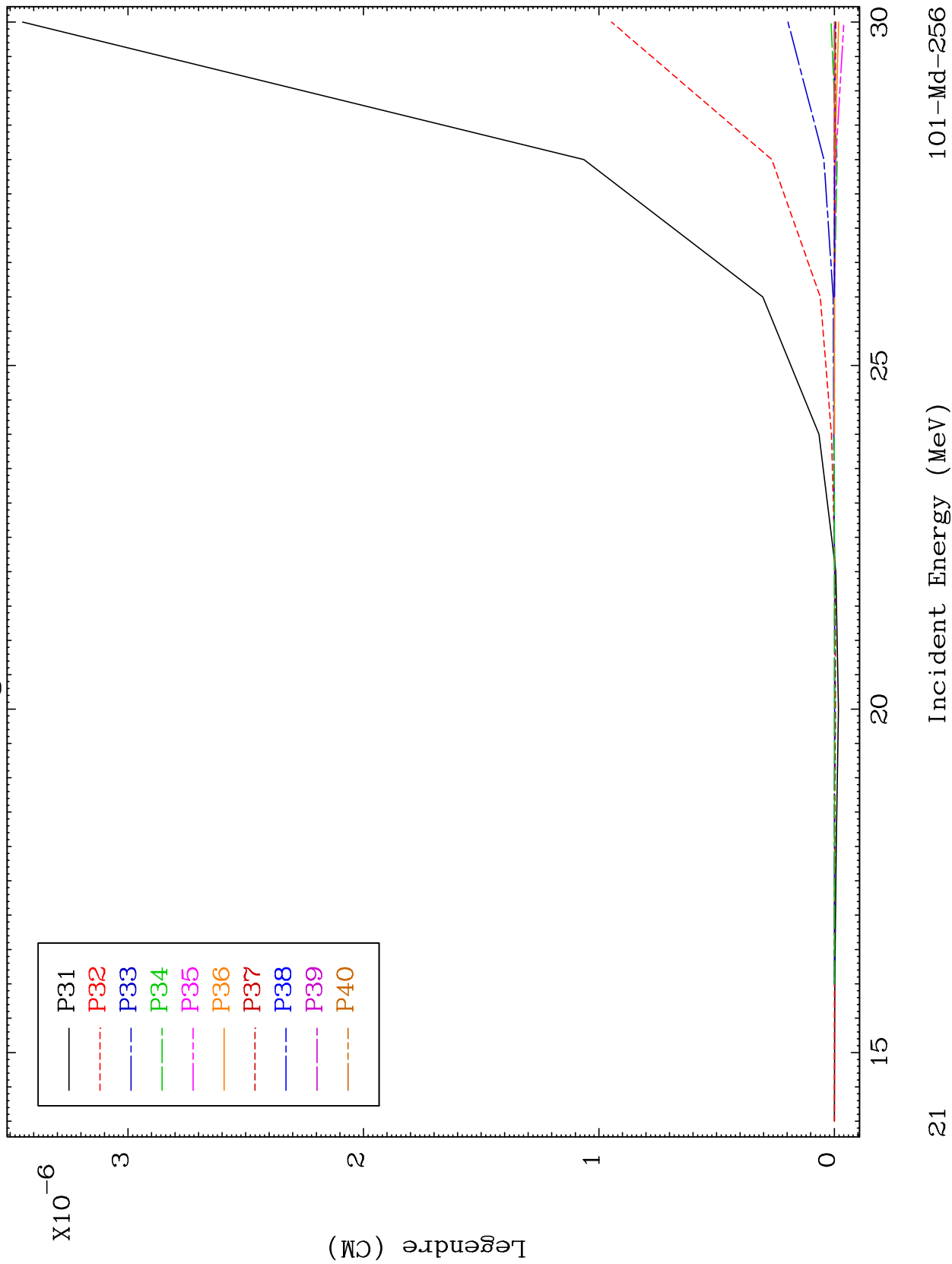
Incident Energy (MeV)

101-Md-256

MAT 99577

Elastic Legendre Coefficients

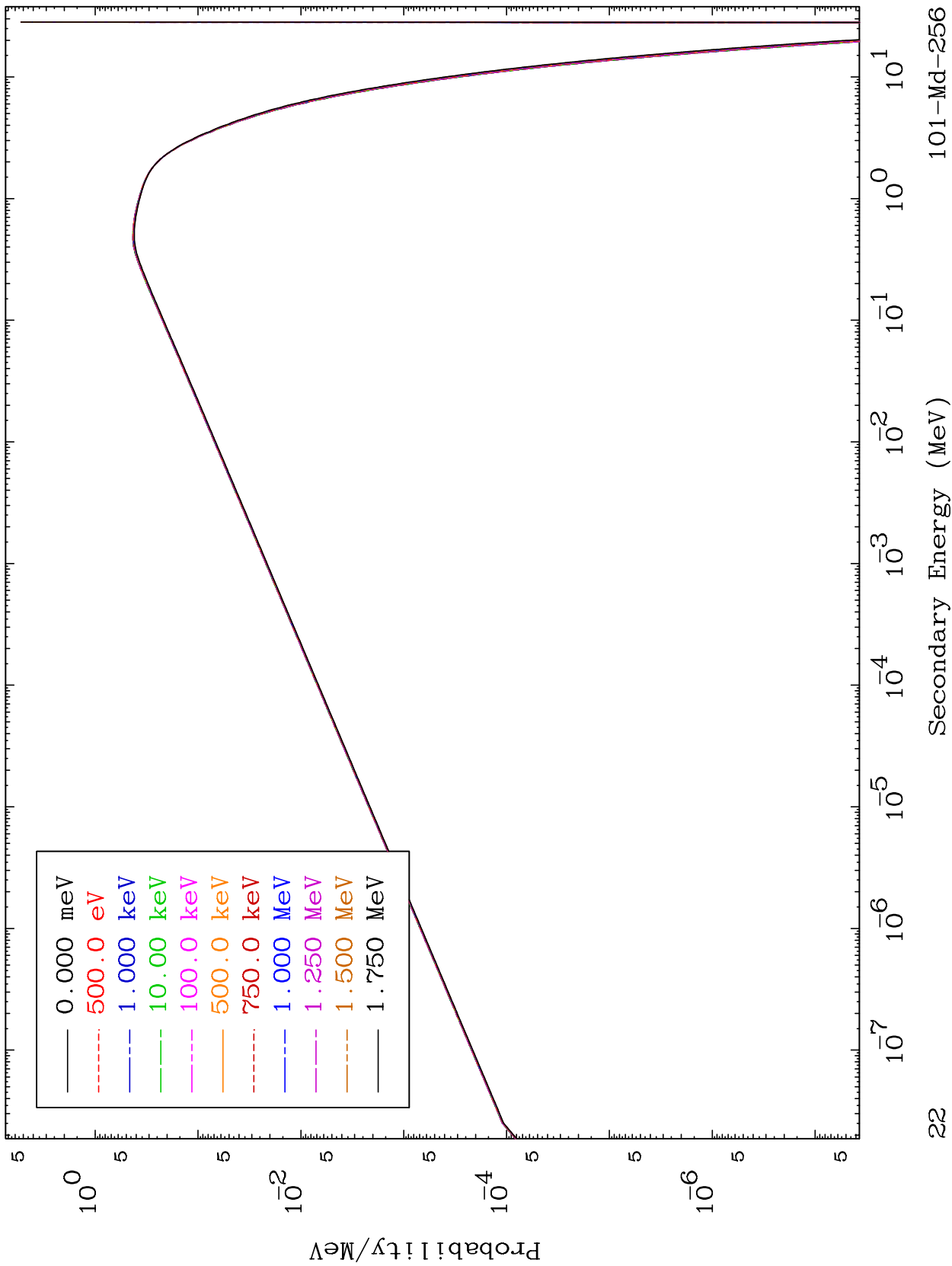
101-Md-256



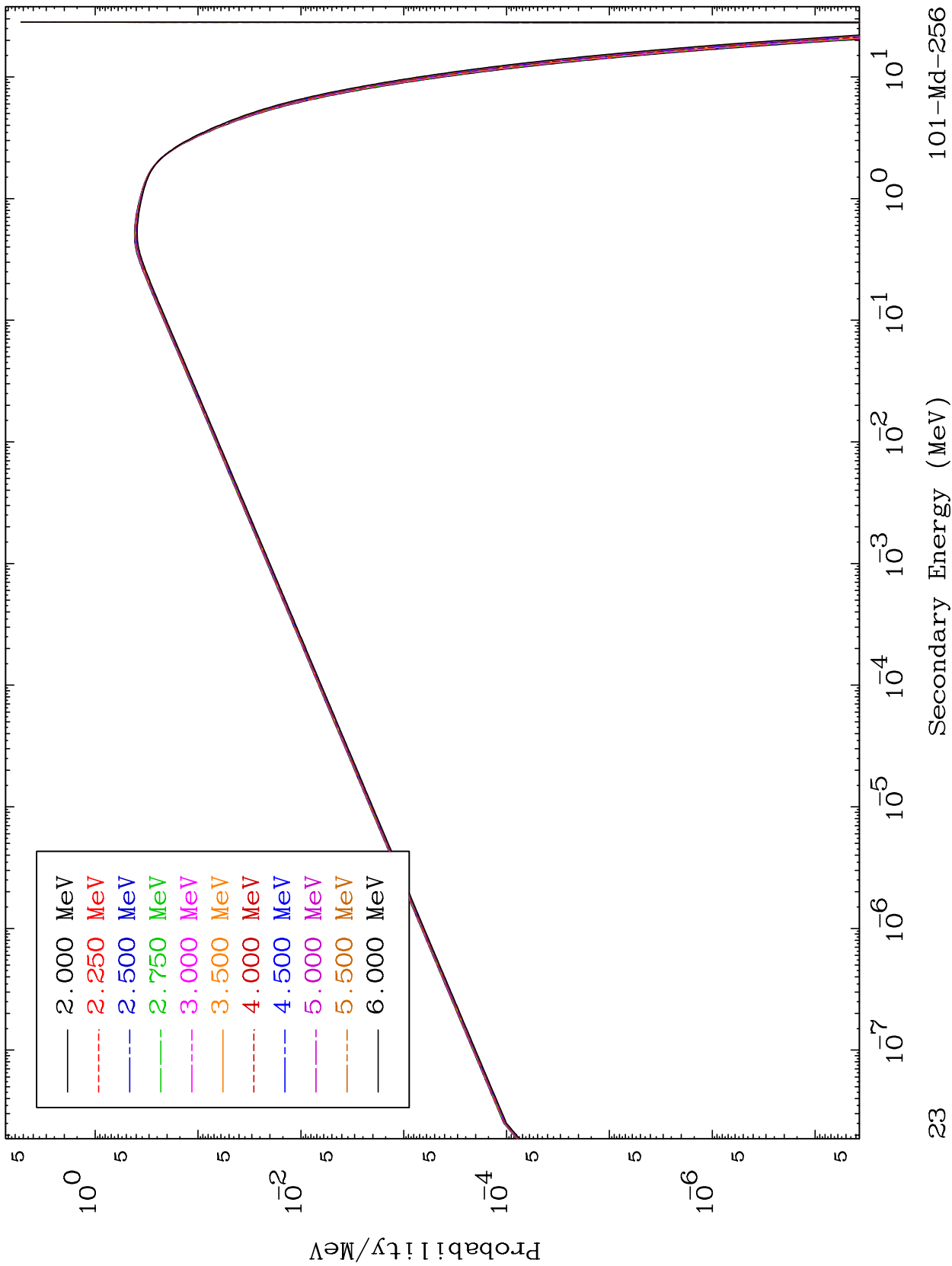
21

Incident Energy (MeV)

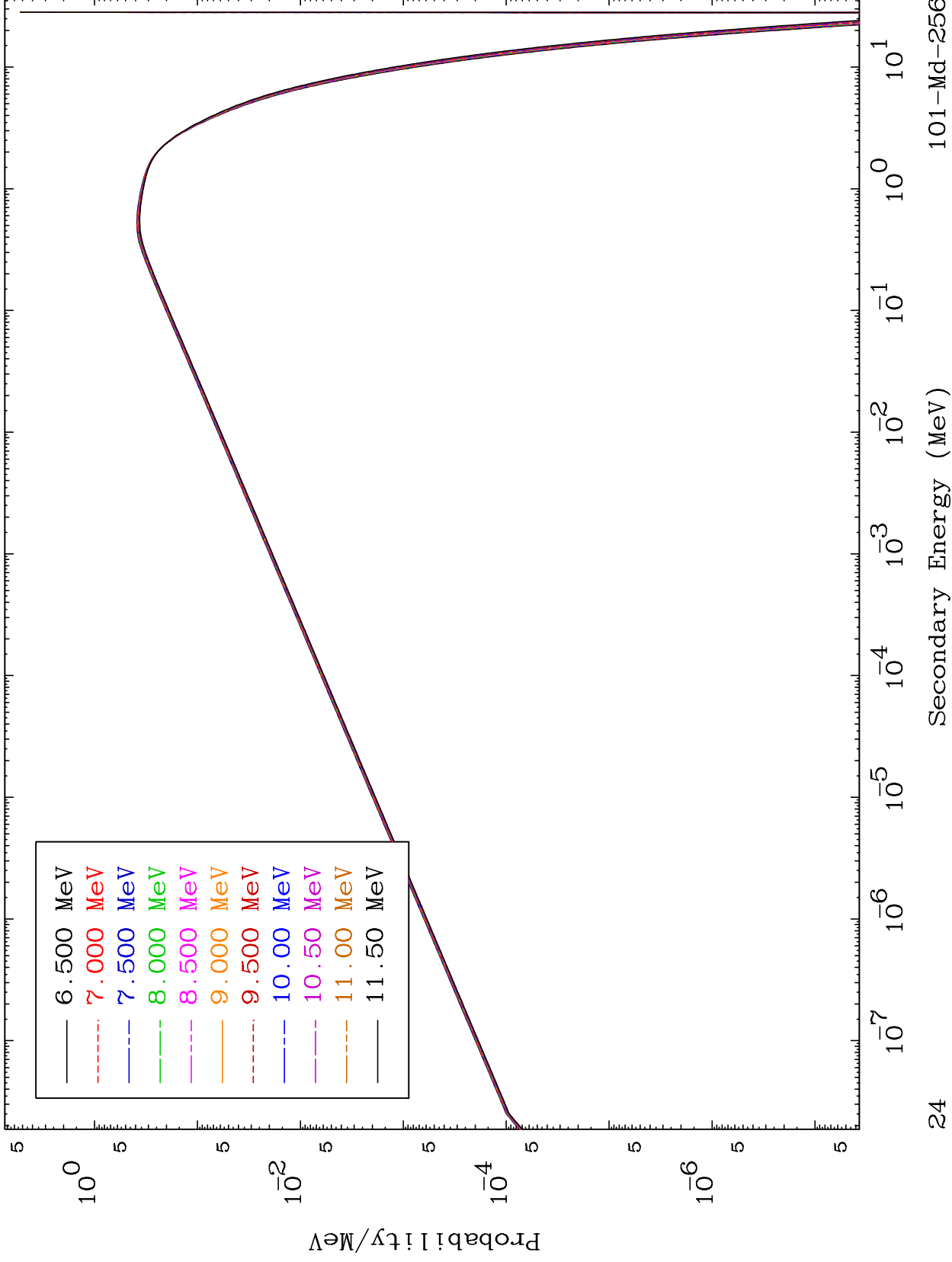
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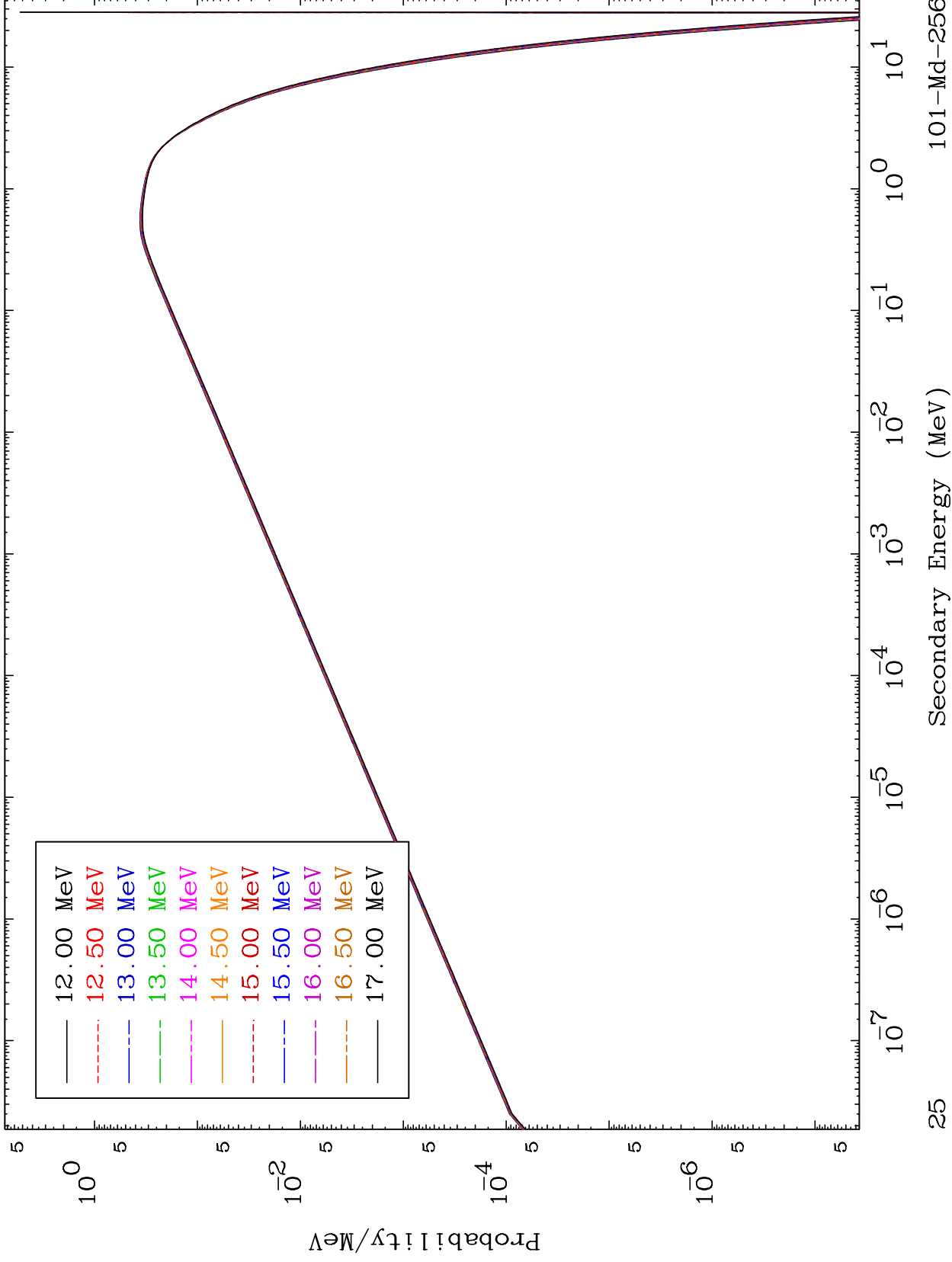


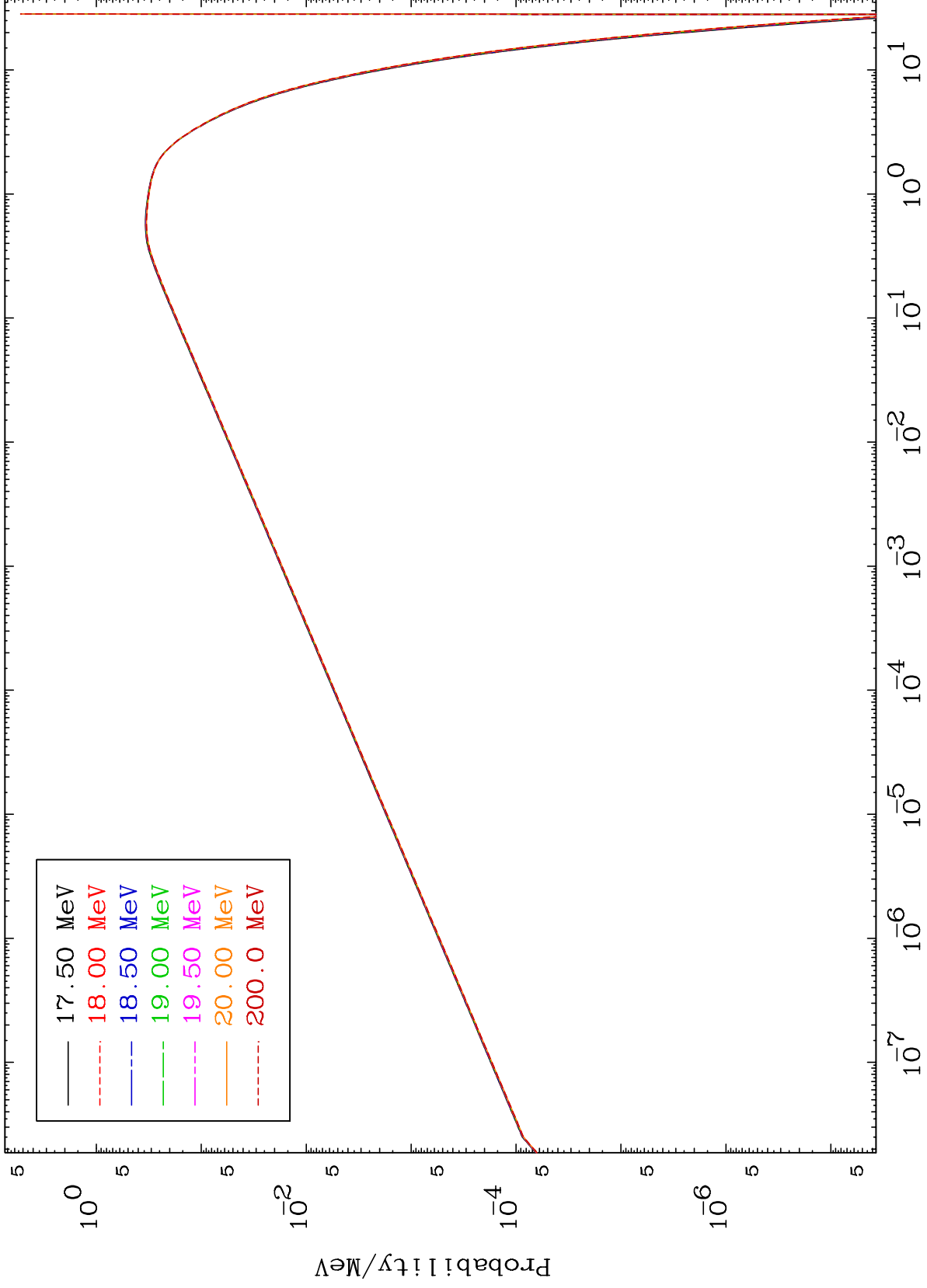
Fission
Energy Distribution



Fission
Energy Distribution



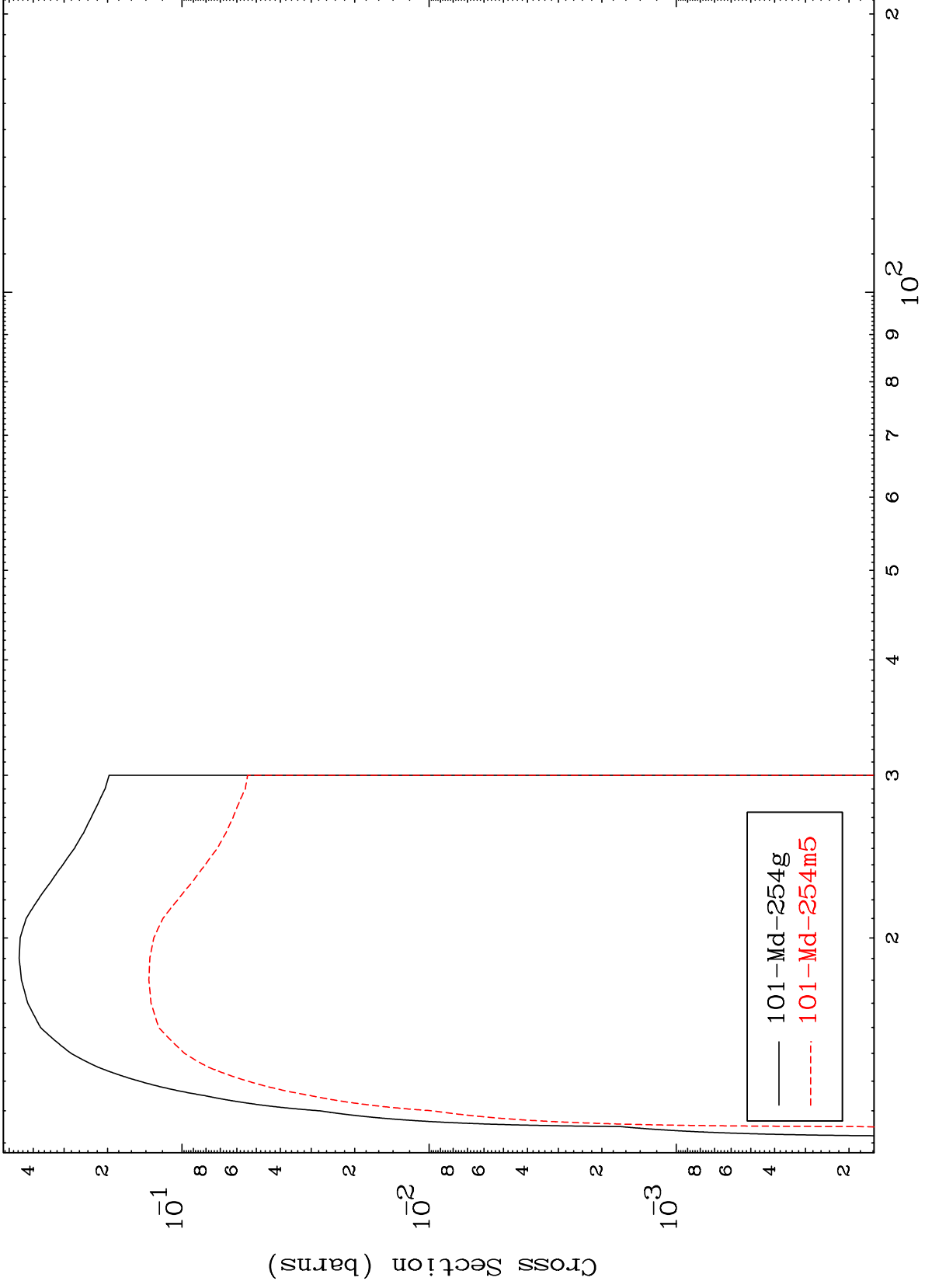




MAT 9957

101-Md-256

(n,3n)
Radionuclide Production Cross Section



101-Md-256

Incident Energy (MeV)

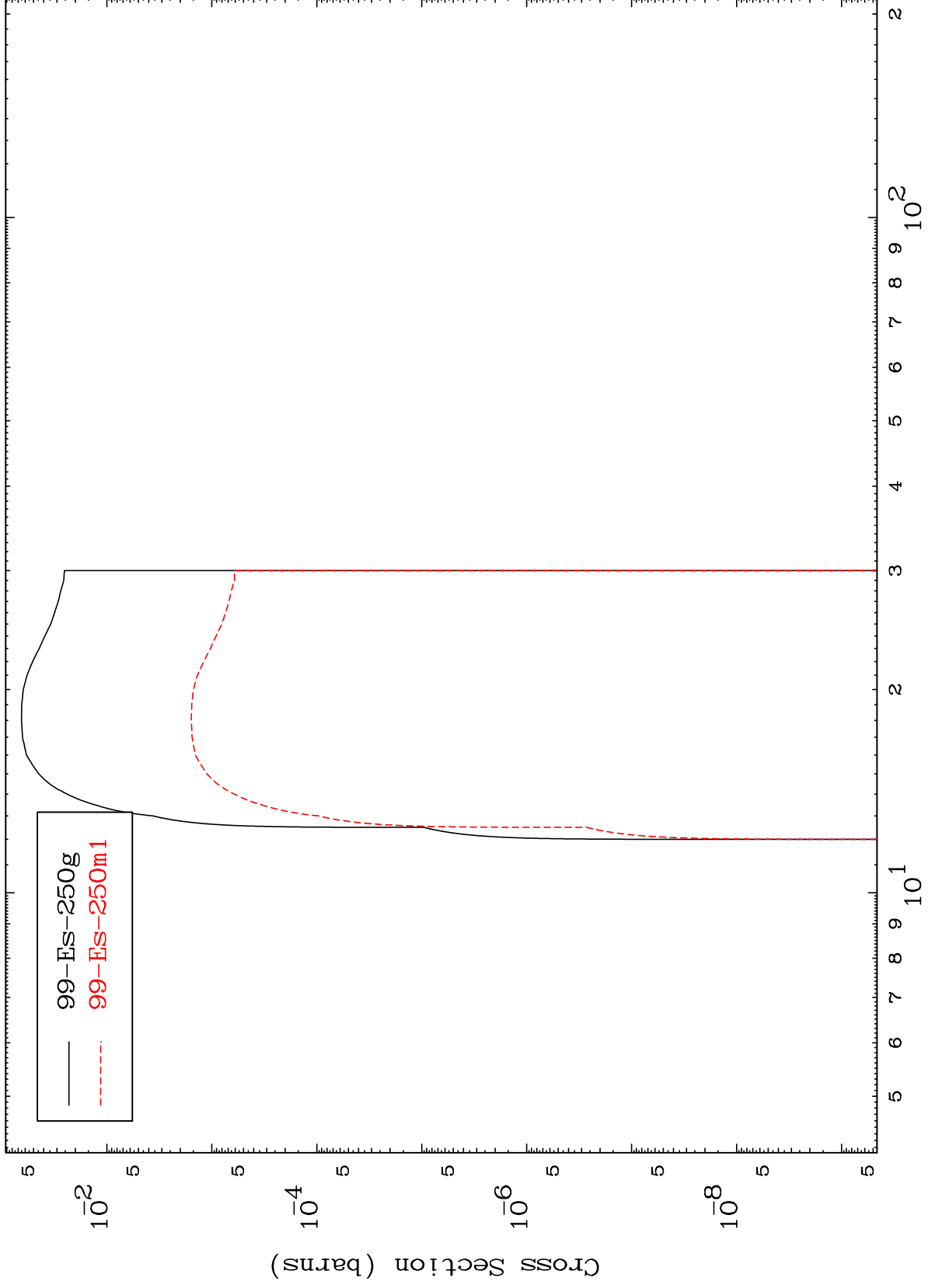
27

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(n,3n) α

101-Md-256

Radionuclide Production Cross Section



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Incident Energy (MeV)

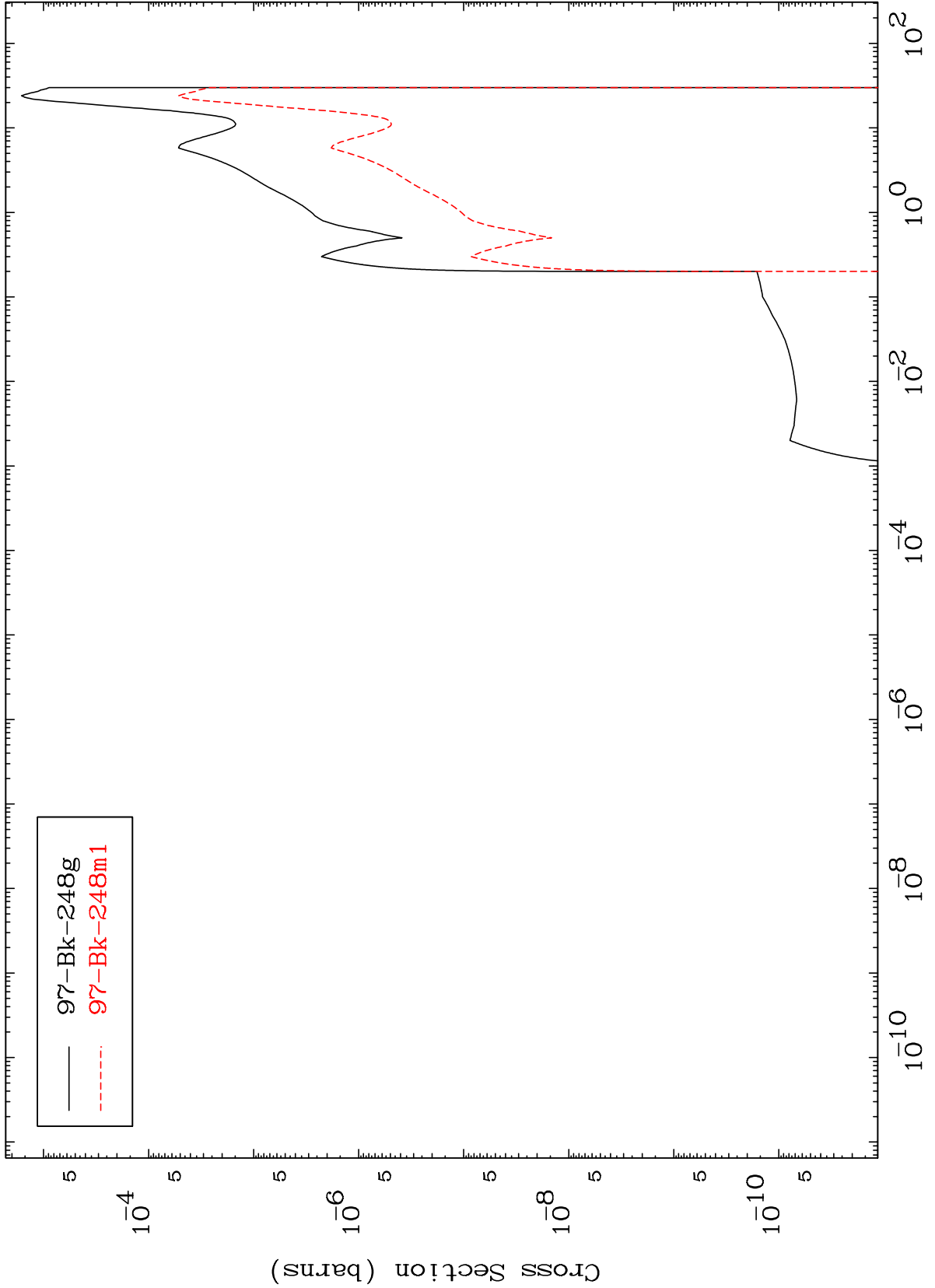
101-Md-256

MAT 9957

(n,n') 2 α

101-Md-256

Radionuclide Production Cross Section

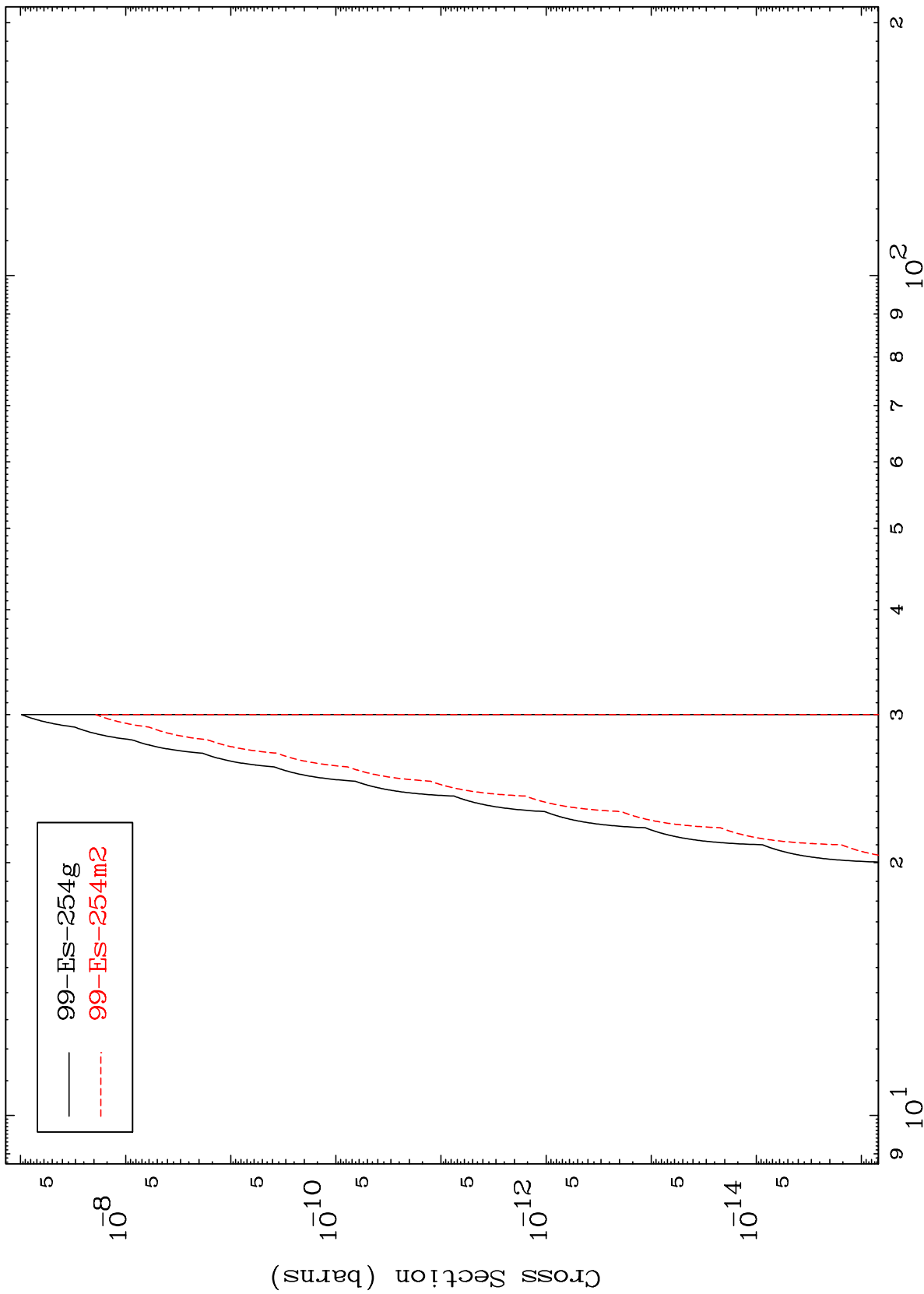


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(n,2n) p

101-Md-256

Radionuclide Production Cross Section



Incident Energy (MeV)

101-Md-256

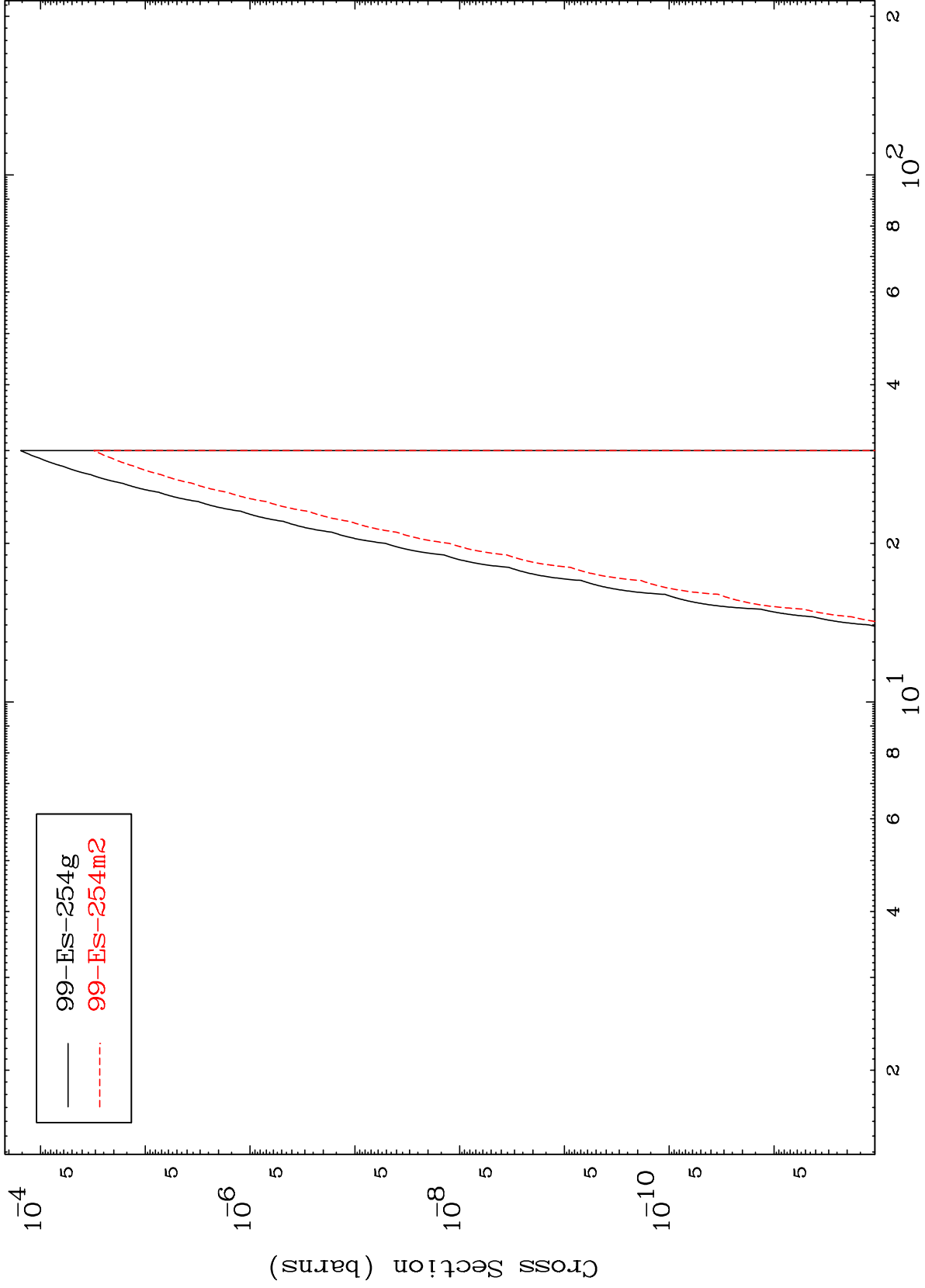
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(n,He-3)

101-Md-256

Radionuclide Production Cross Section



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Incident Energy (MeV)

101-Md-256