

Program EVALPLOT
(Version 2015-2)

by

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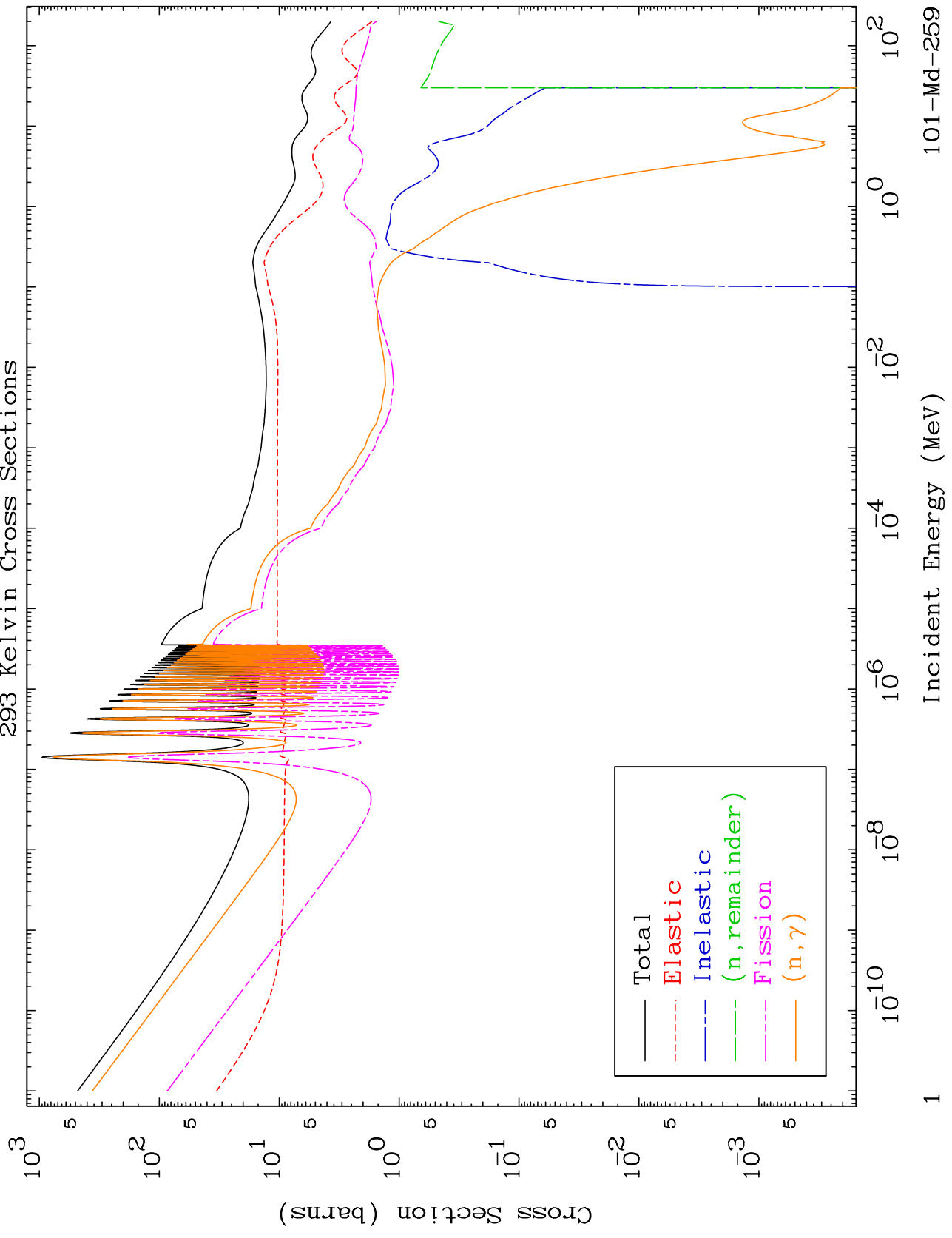
Press Mouse Button to Start

MAT 9960

Major

293 Kelvin Cross Sections

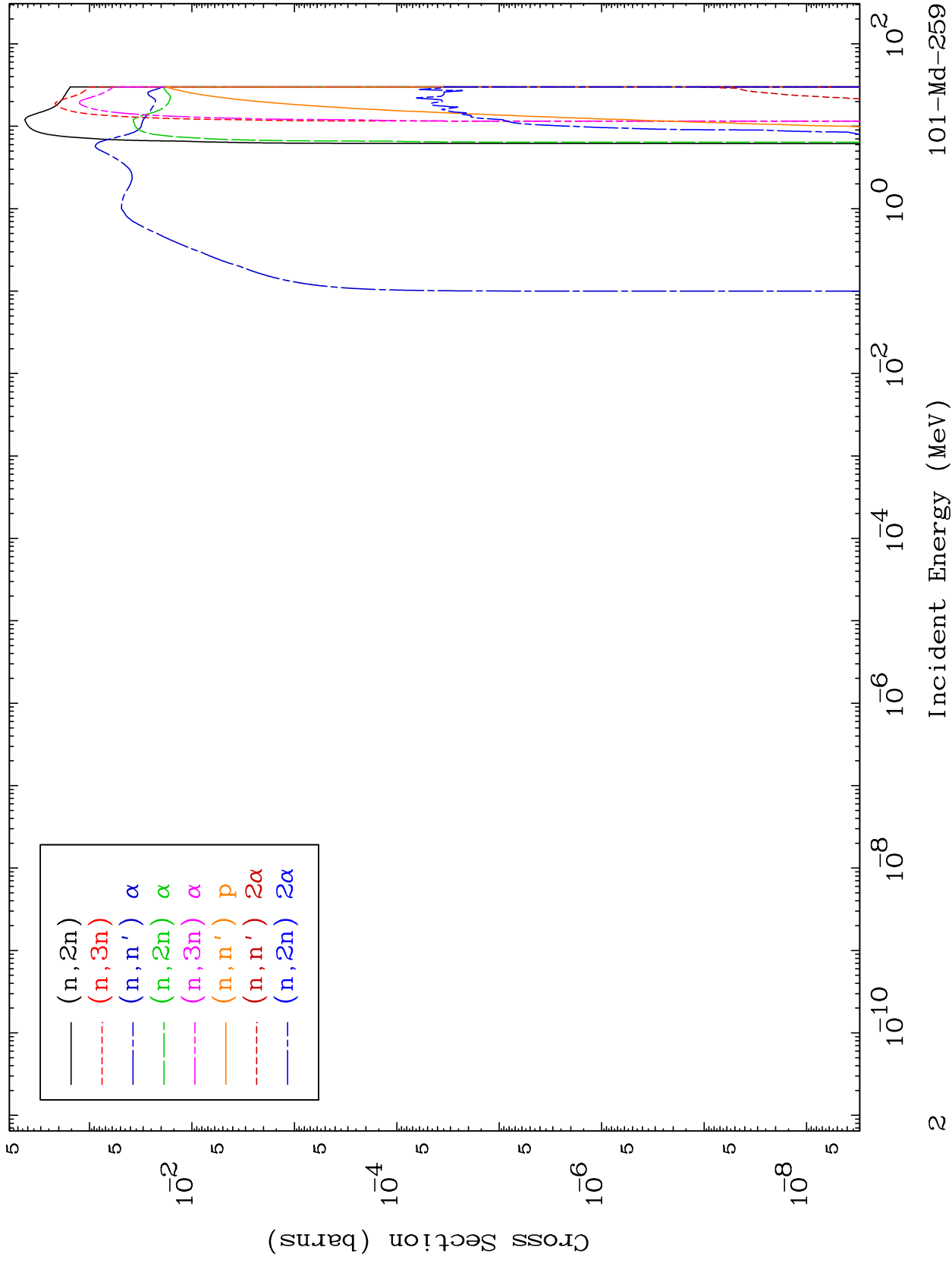
101-Md-259



MAT 9960

Neutron Production
293 Kelvin Cross Sections

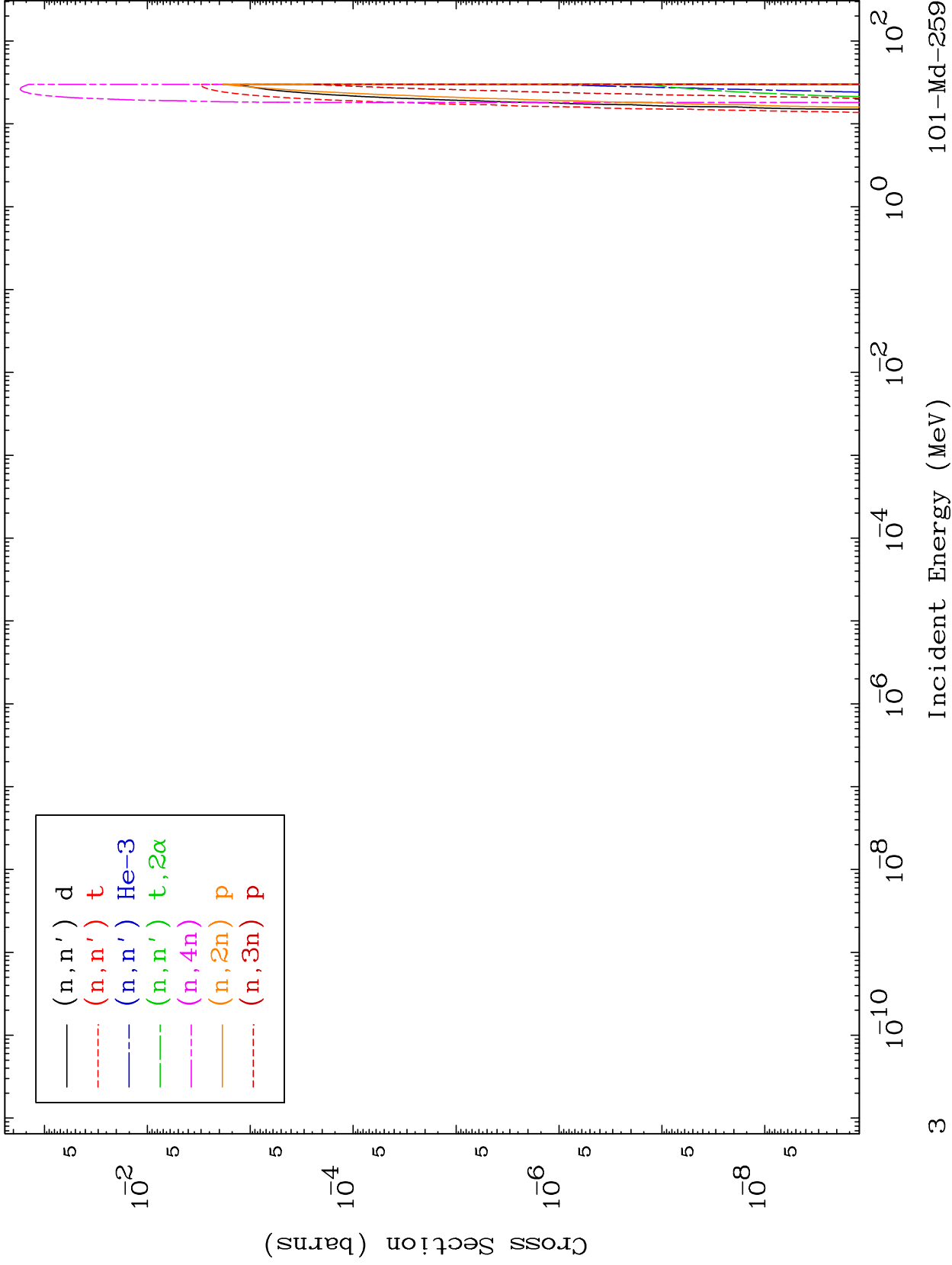
101-Md-259



MAT 9960

Neutron Production
293 Kelvin Cross Sections

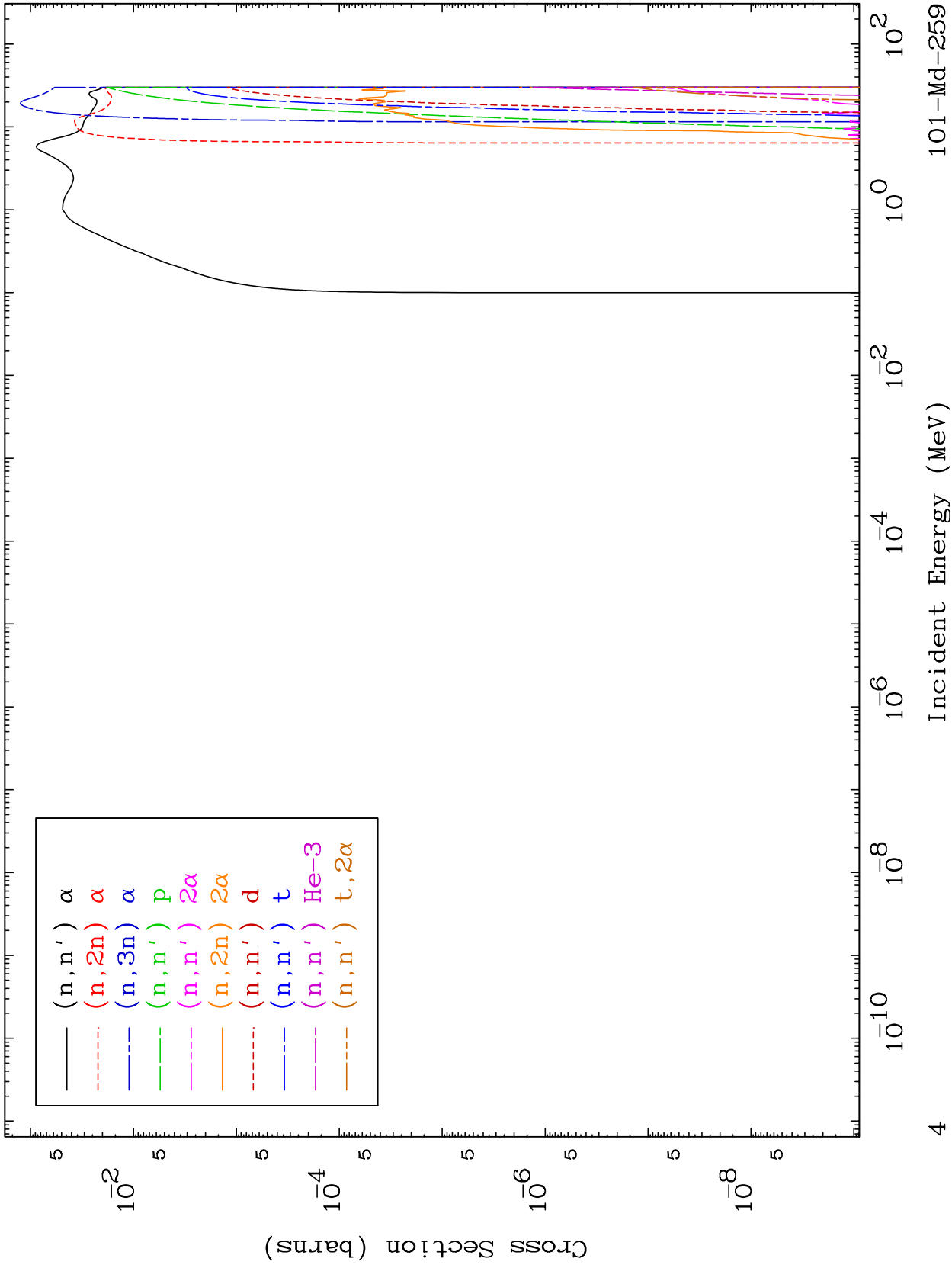
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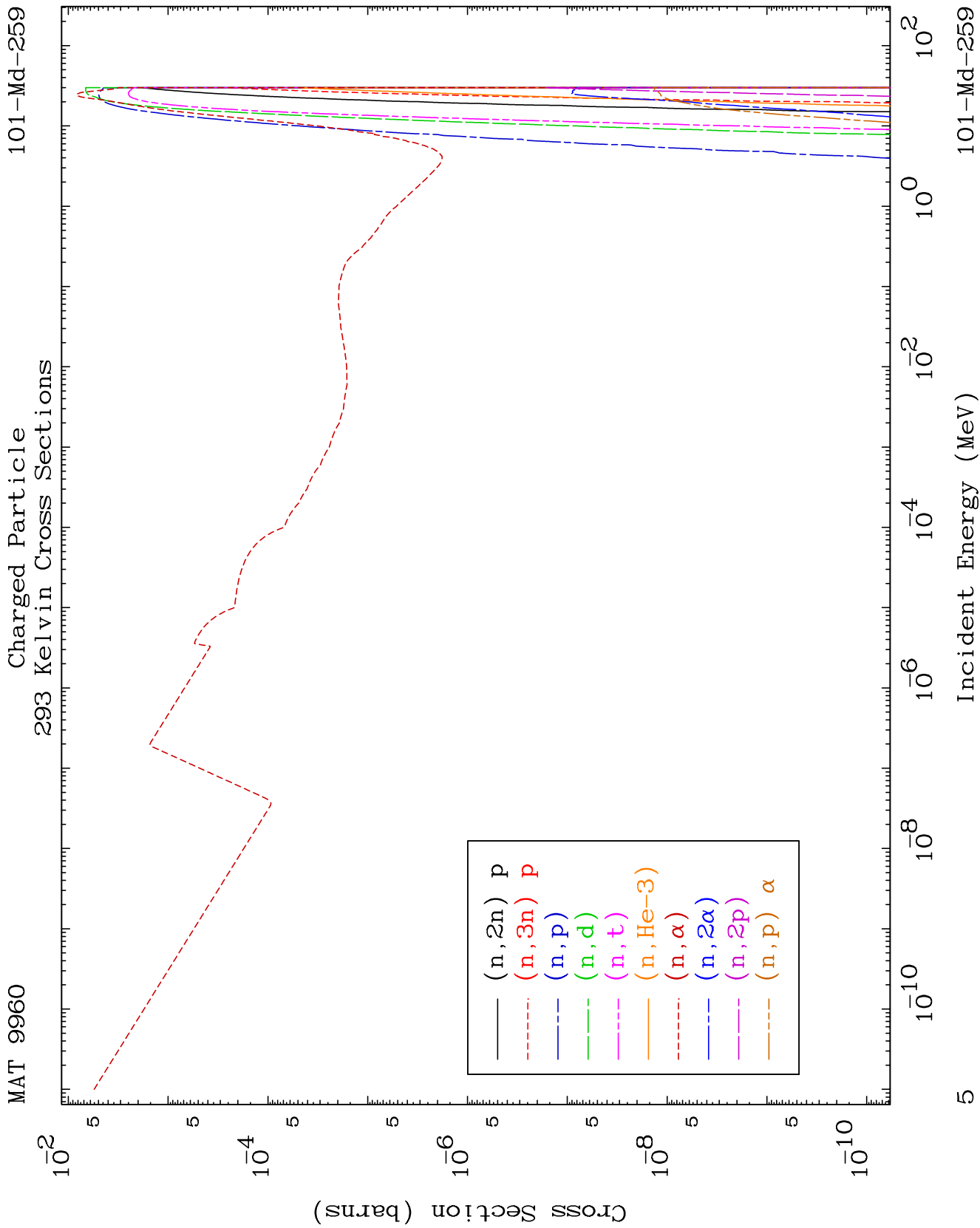


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Charged Particle
293 Kelvin Cross Sections

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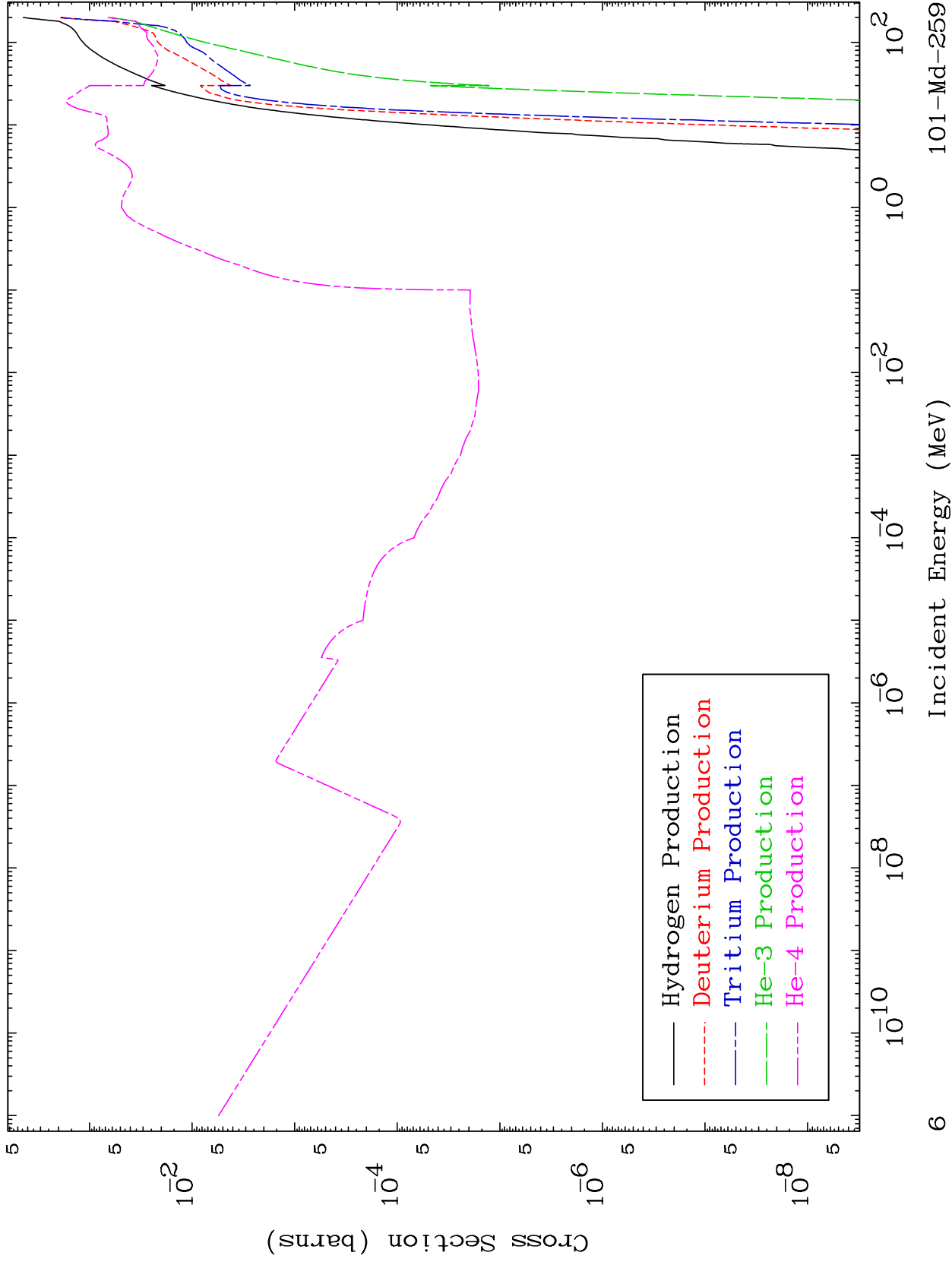


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Particle Production
293 Kelvin Cross Sections

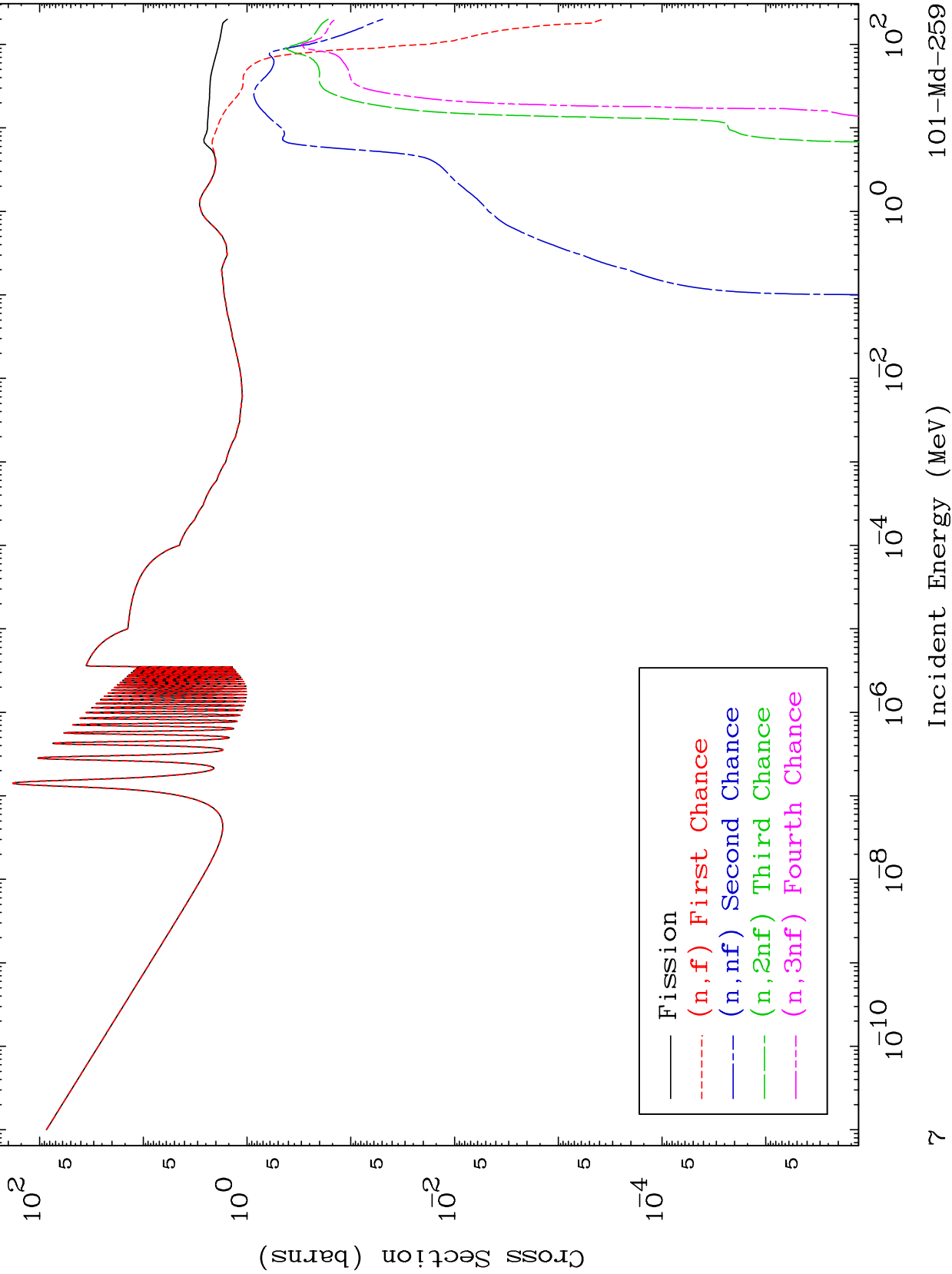
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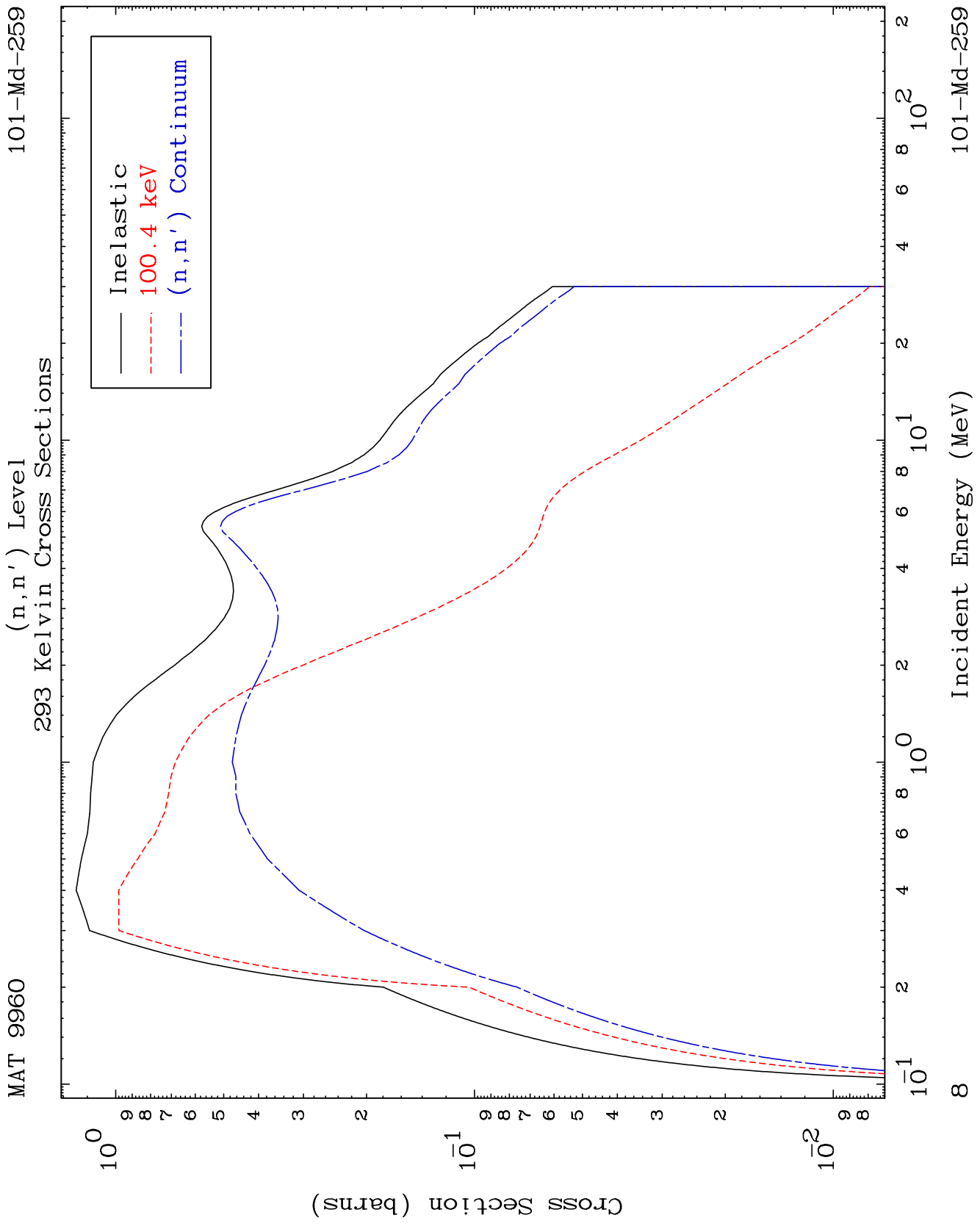


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Fission
293 Kelvin Cross Sections

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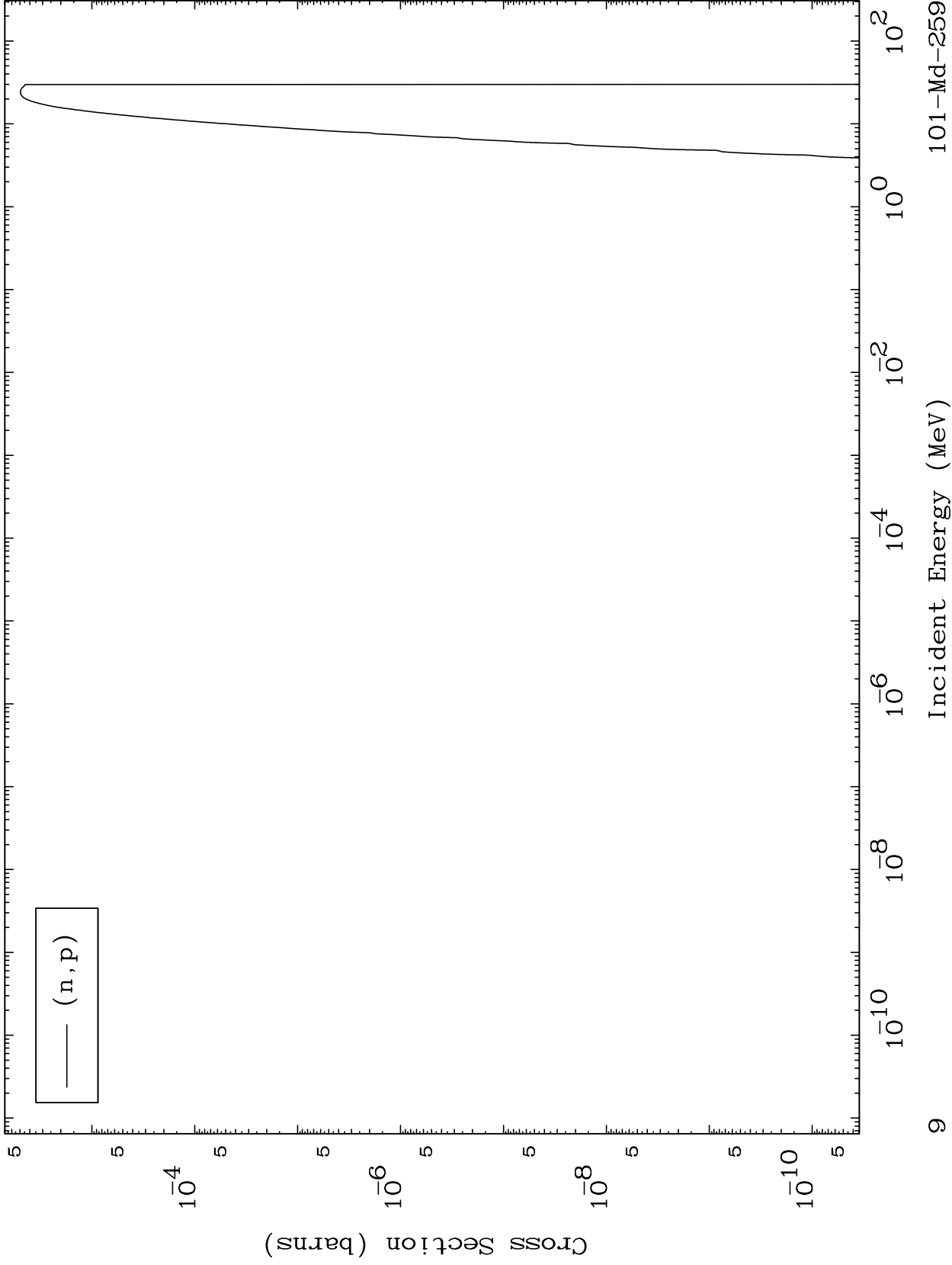




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(n,p) Levels
293 Kelvin Cross Sections

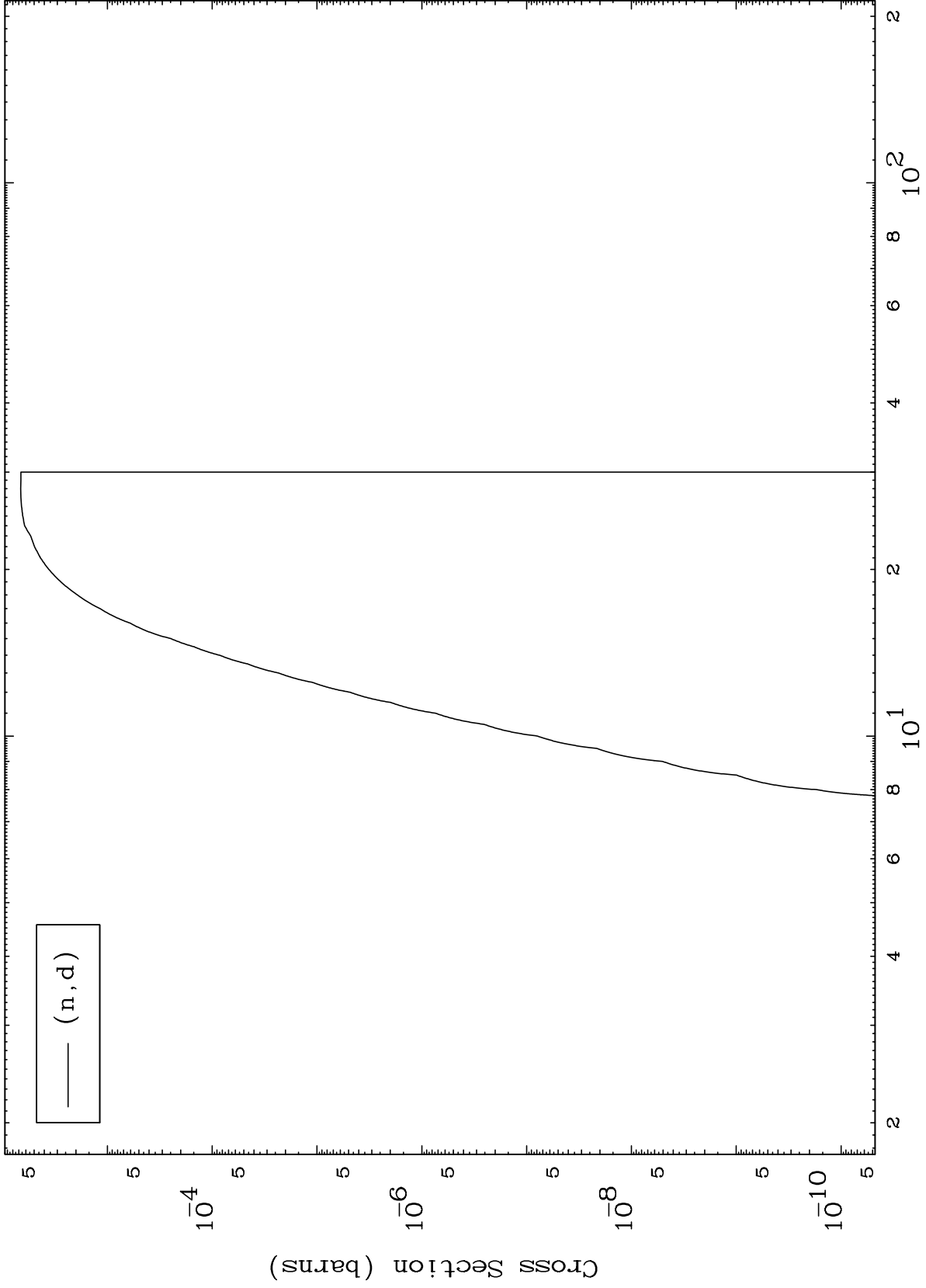
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(n,d) Levels
293 Kelvin Cross Sections

101-Md-259



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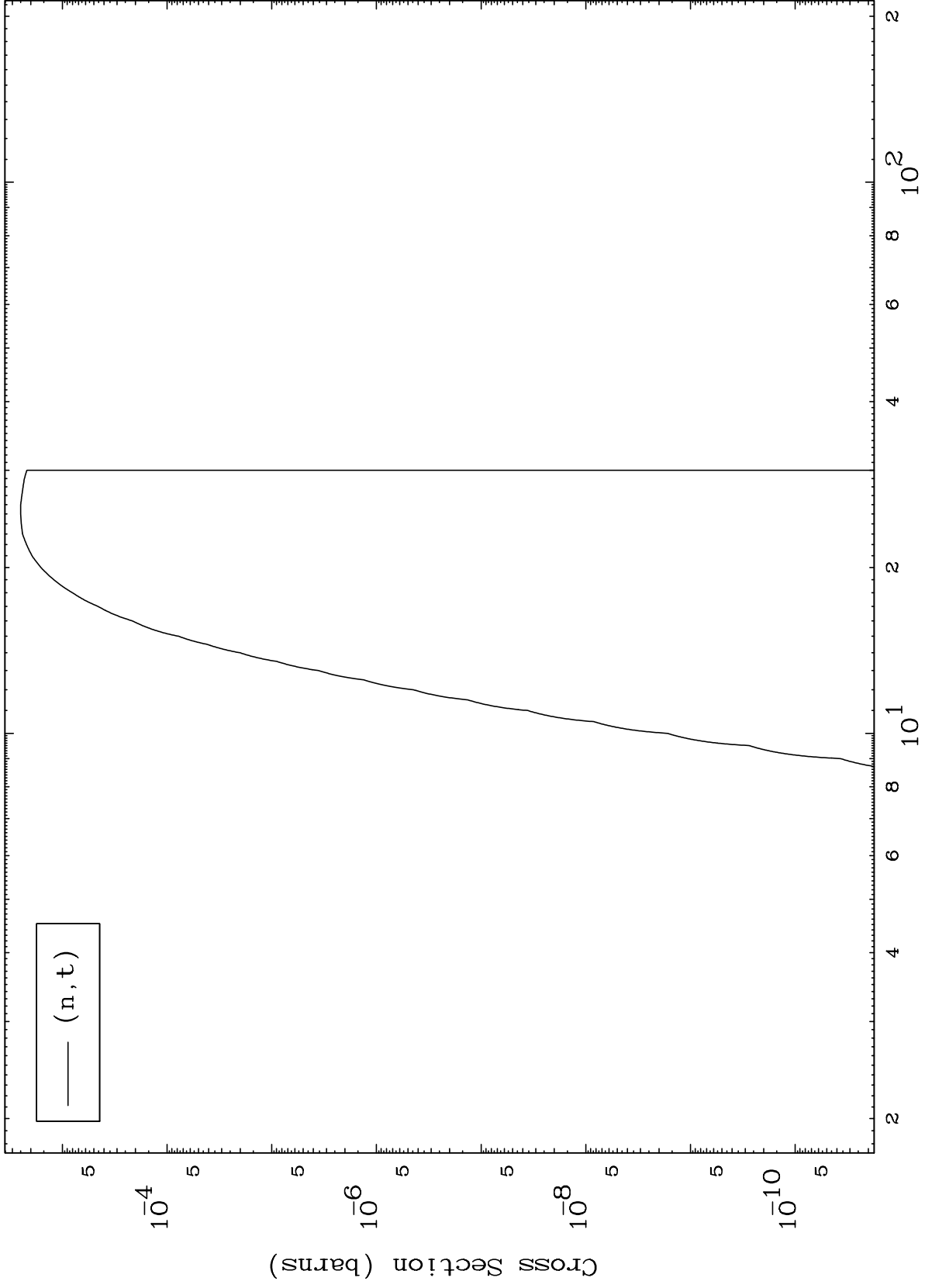
Incident Energy (MeV)

101-Md-259

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(n,t) Levels
293 Kelvin Cross Sections

101-Md-259



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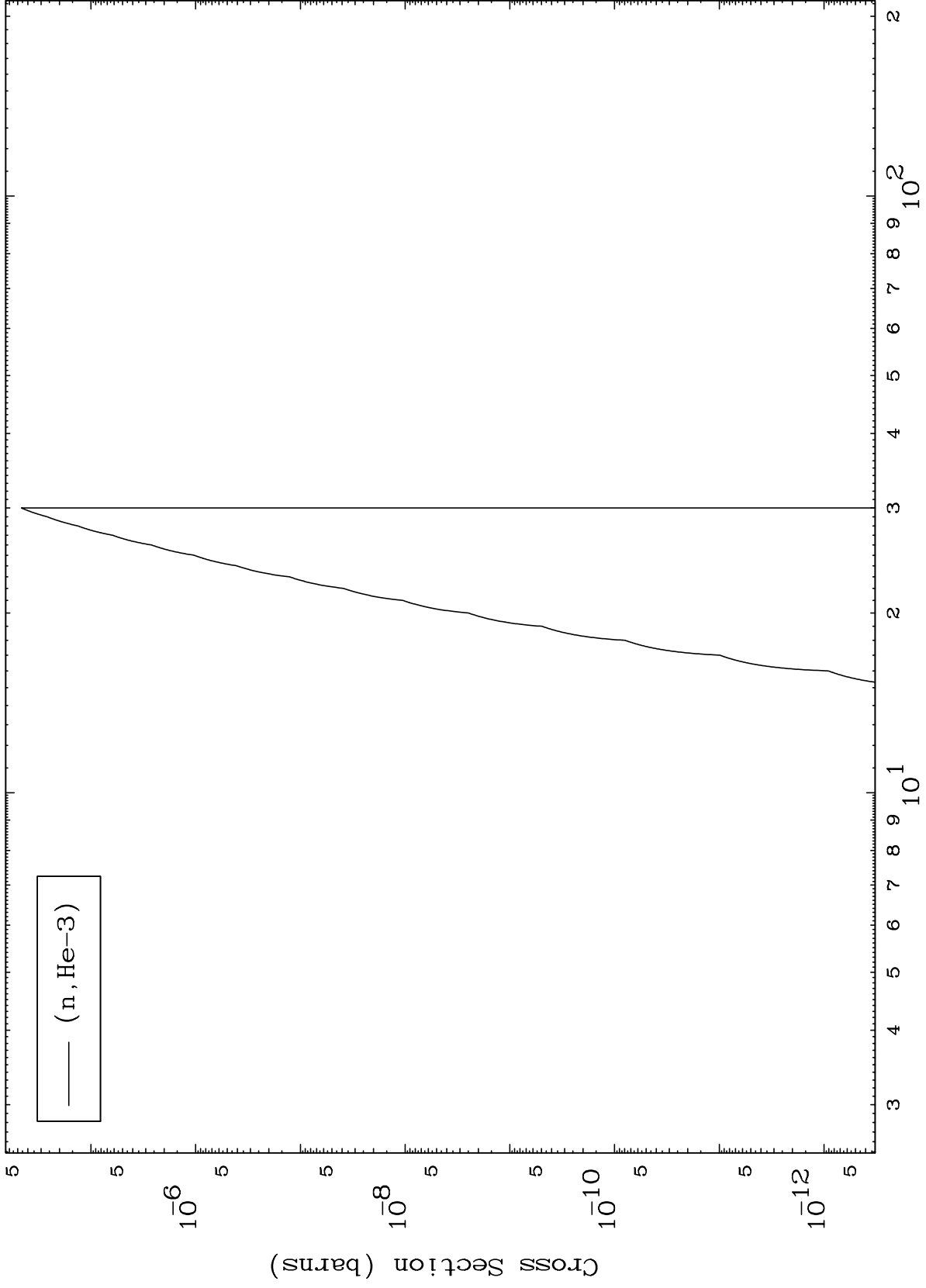
Incident Energy (MeV)

101-Md-259

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(n,He3) Levels
293 Kelvin Cross Sections

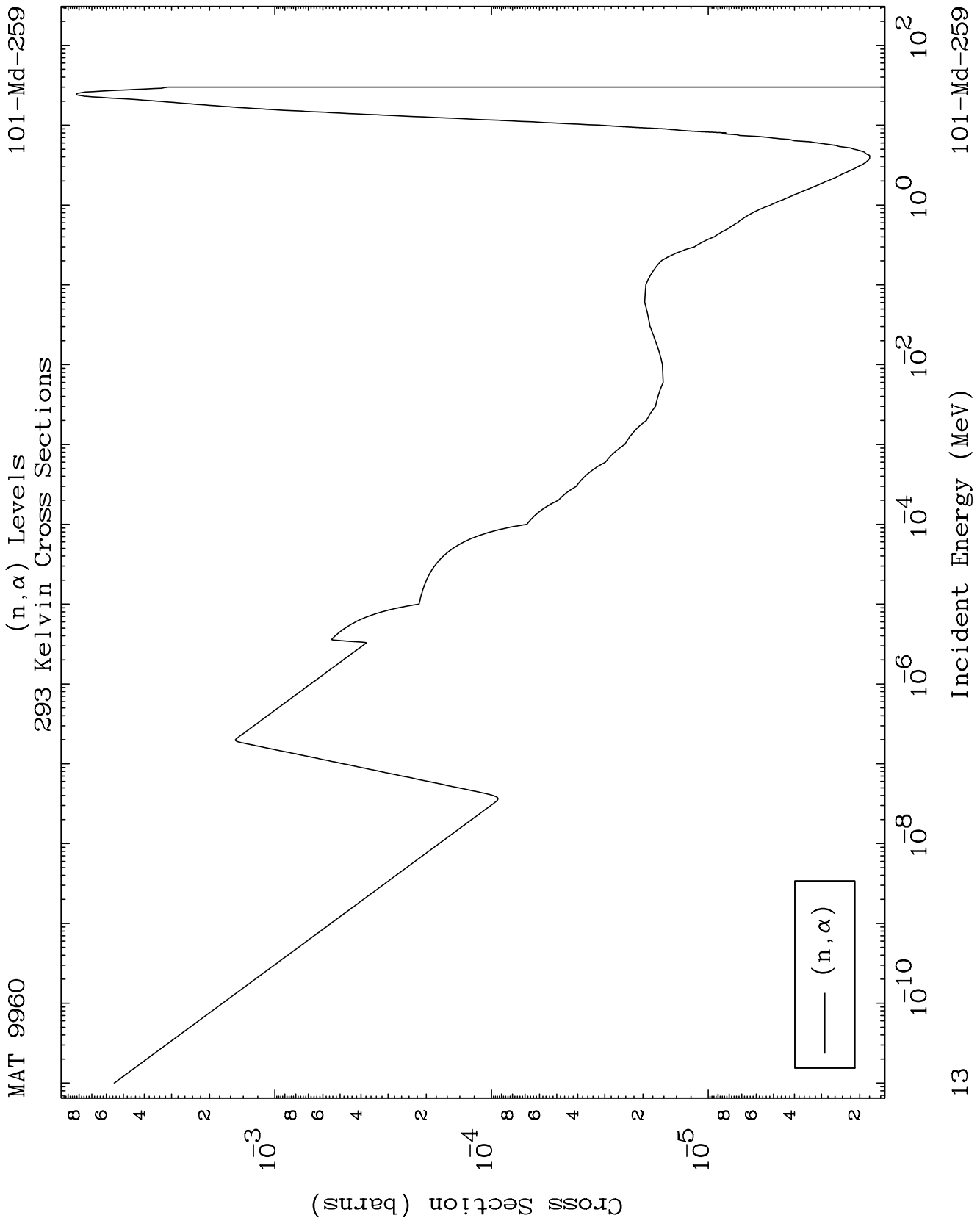
101-Md-259



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Incident Energy (MeV)

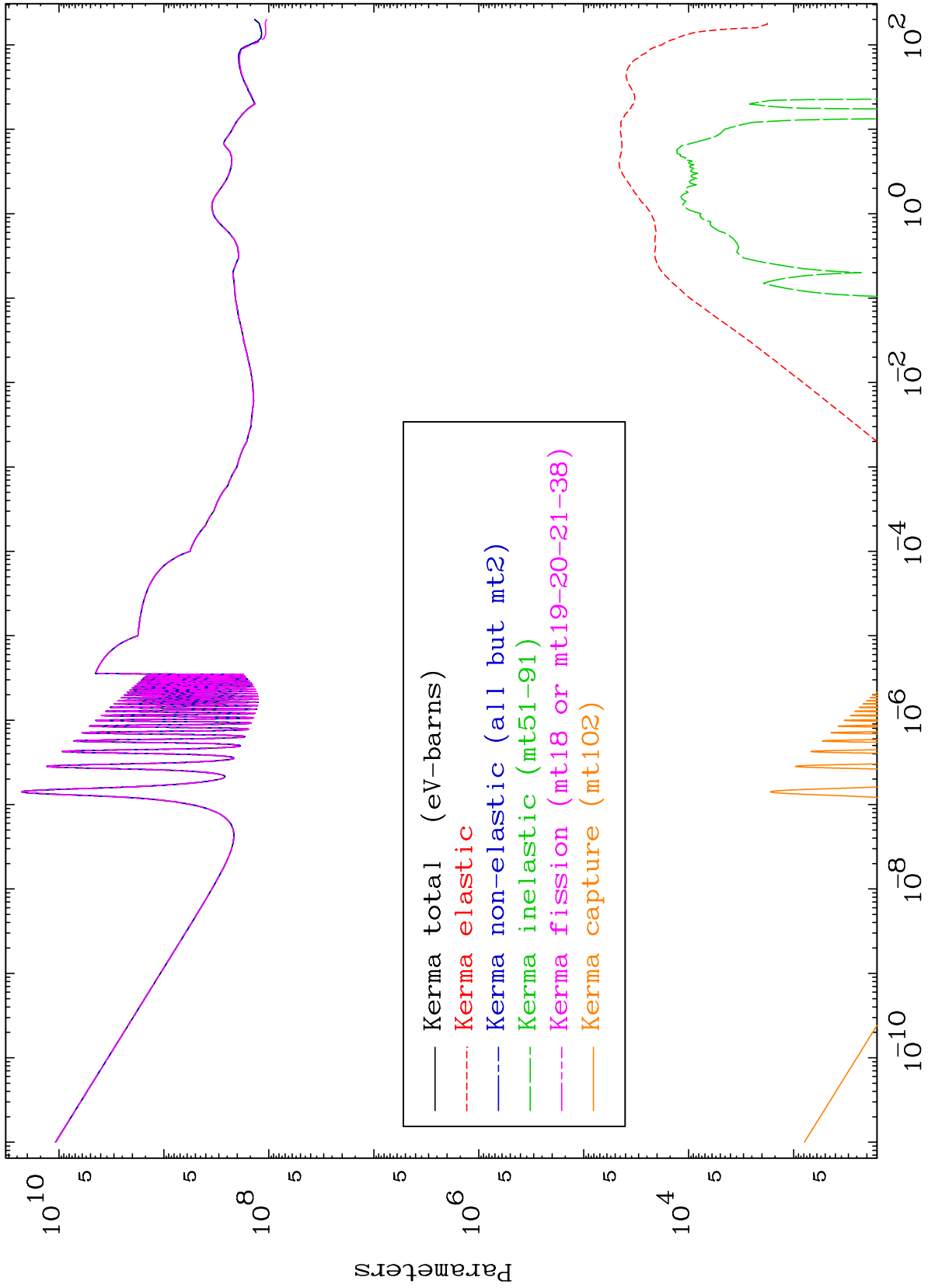
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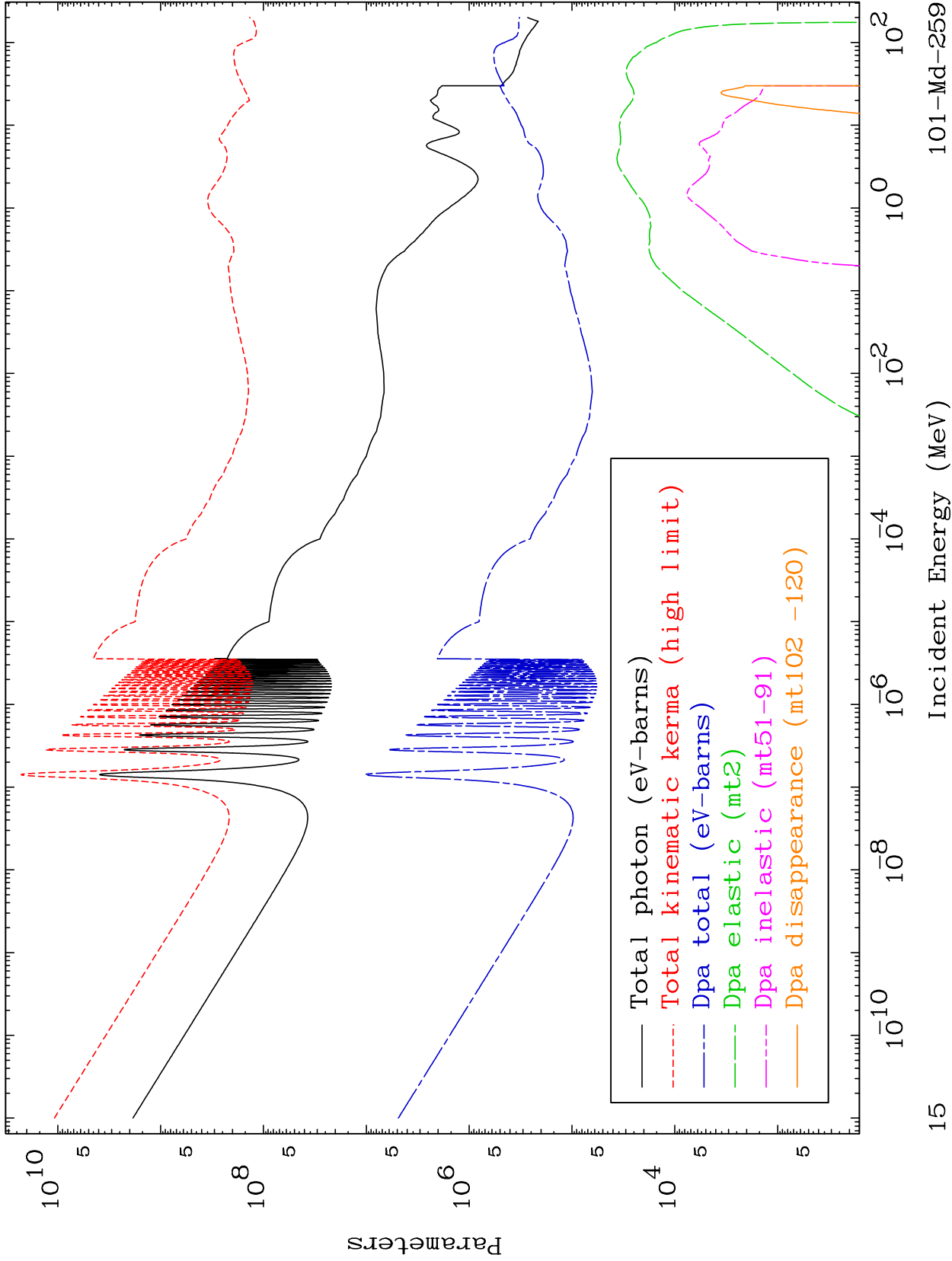


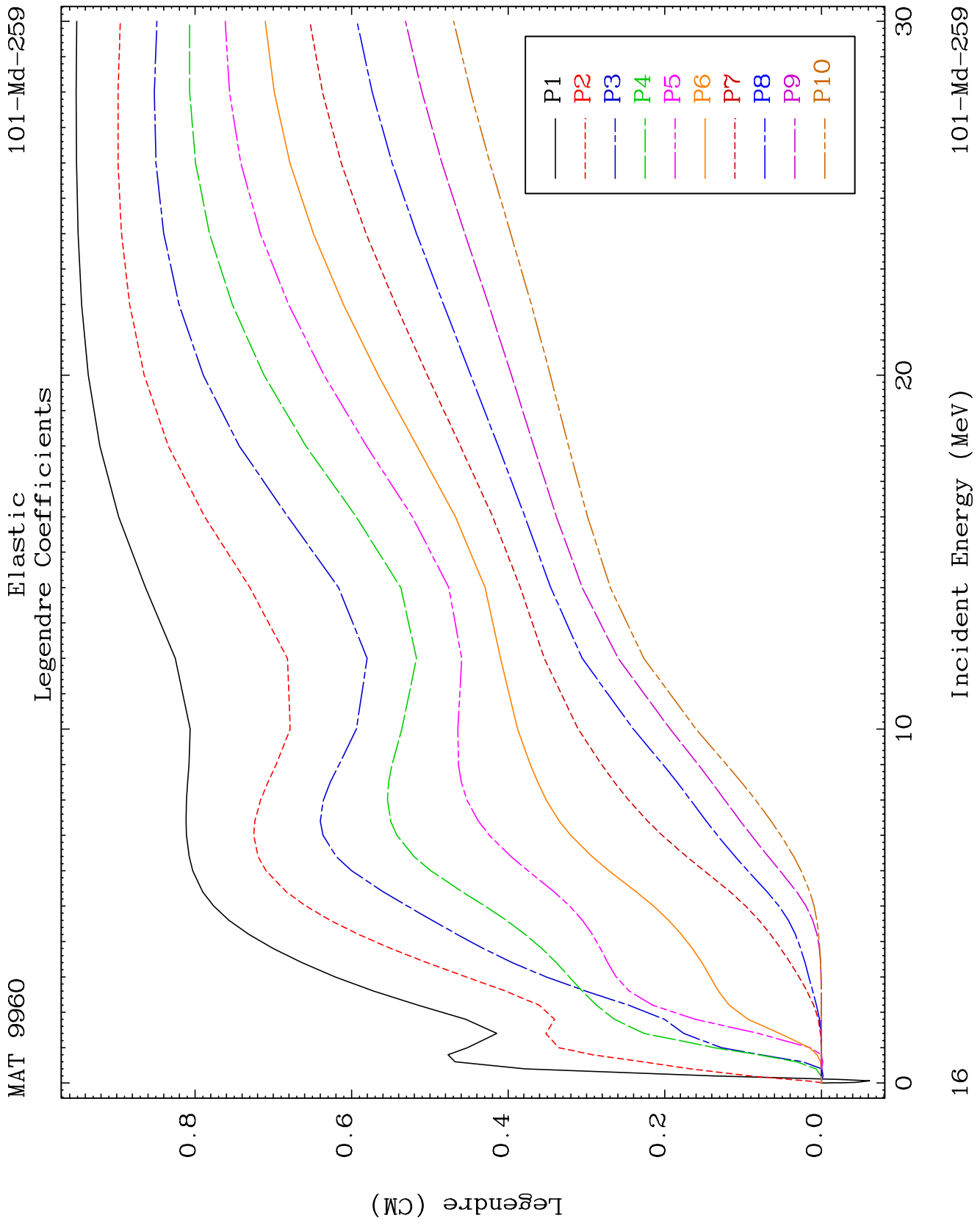
MAT 9960

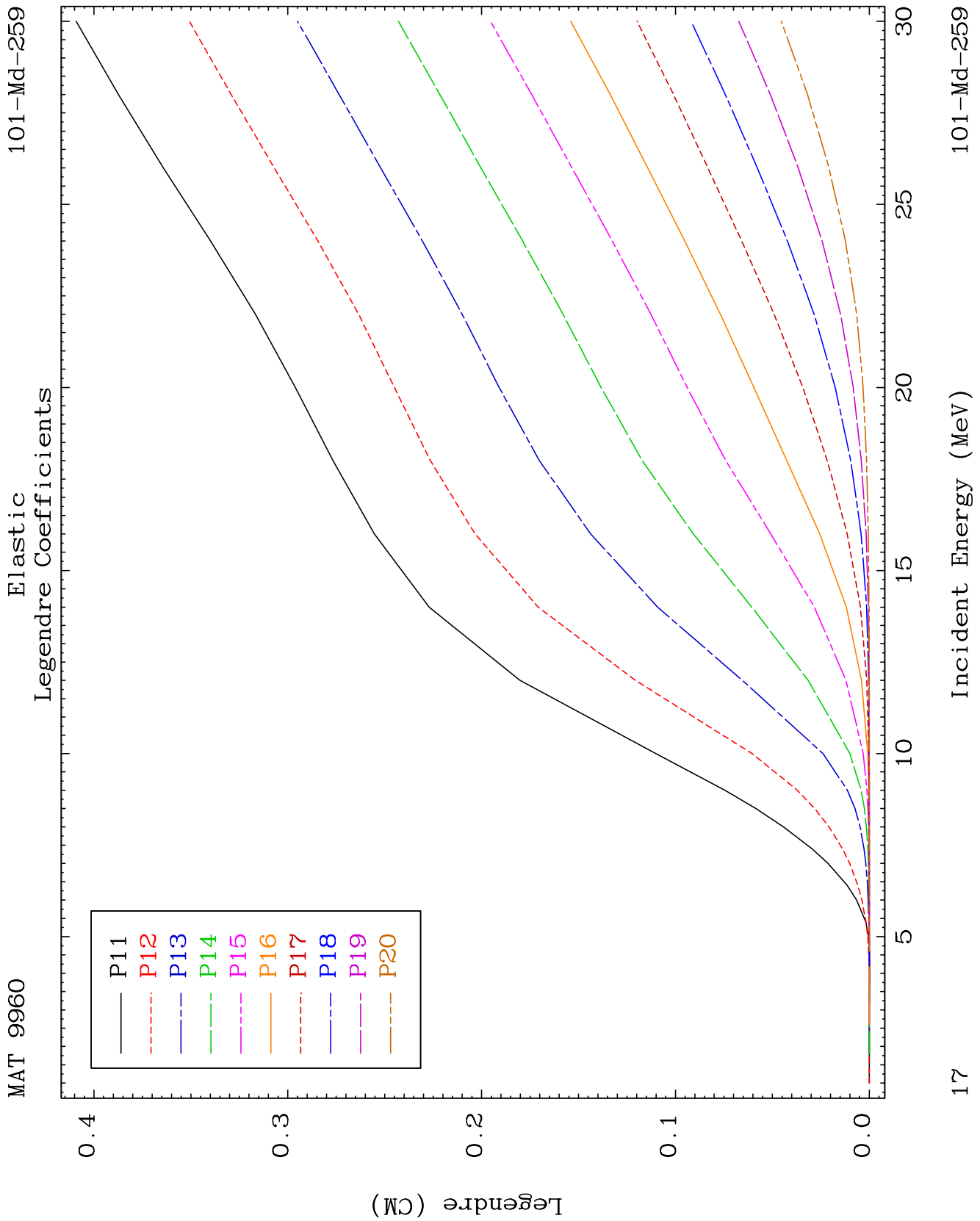
Energy Release
Parameters

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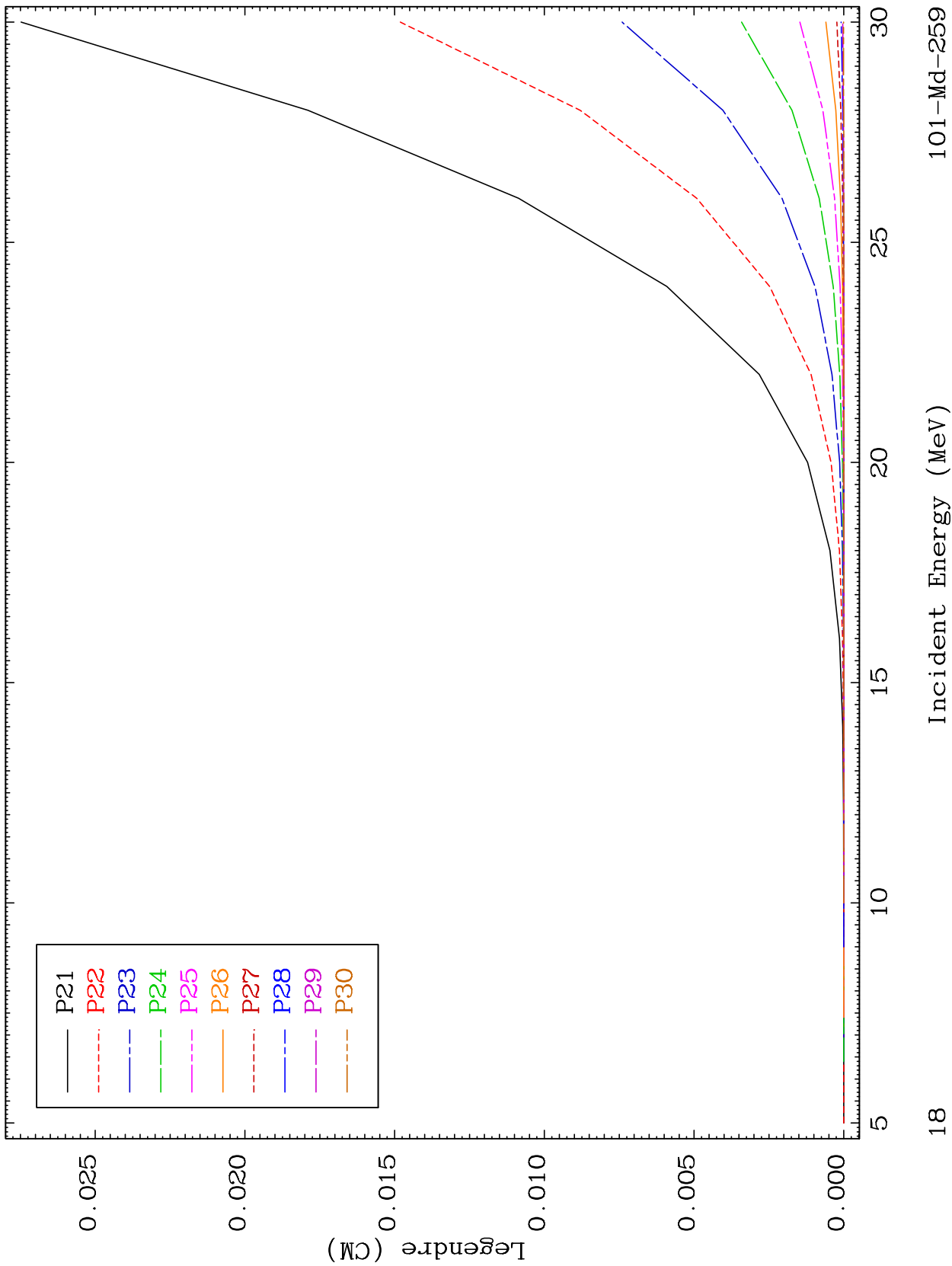




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Elastic Legendre Coefficients

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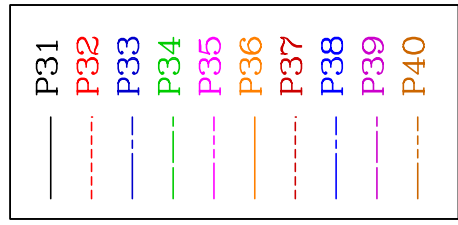
Incident Energy (MeV)

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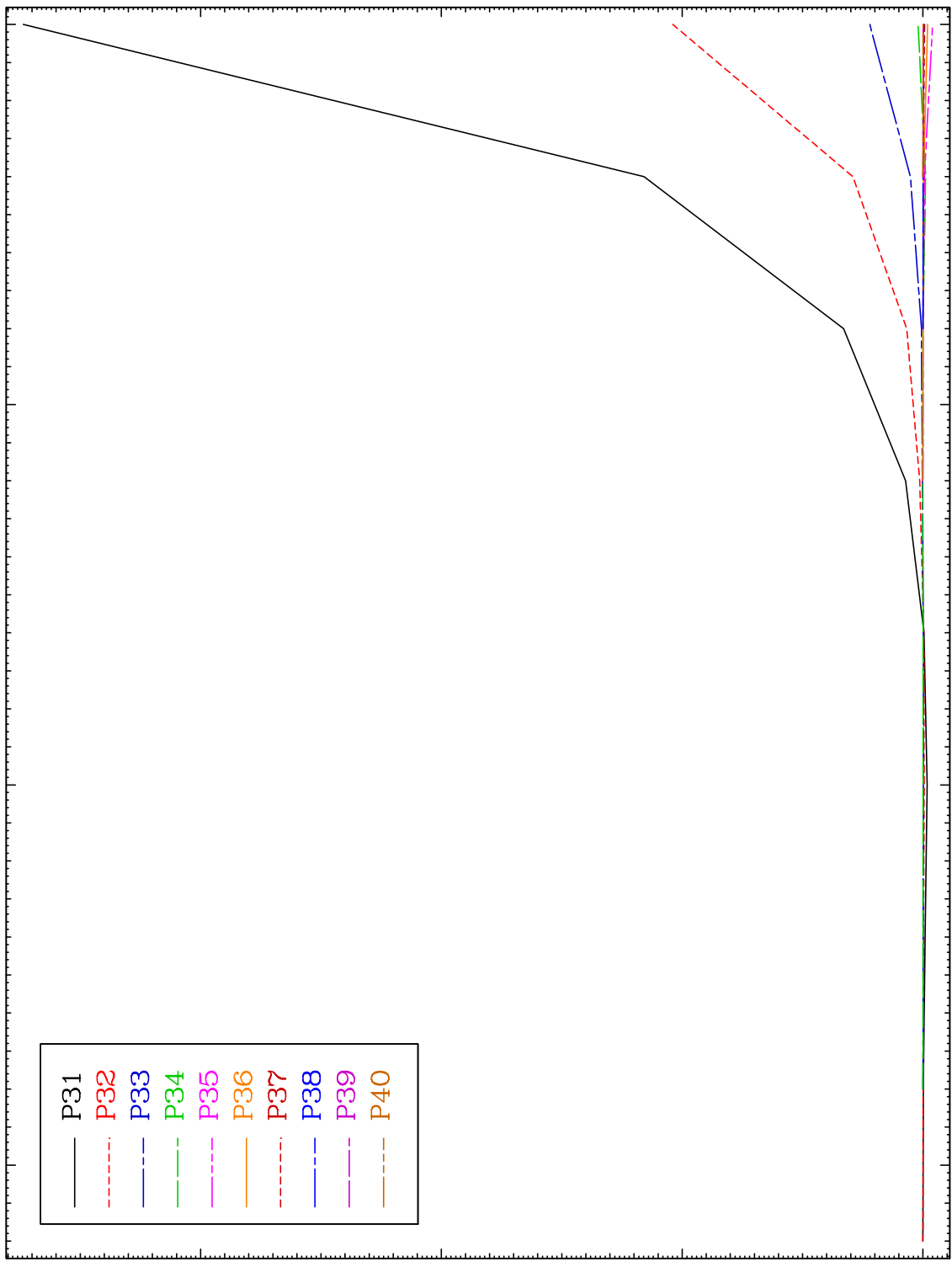
Elastic Legendre Coefficients

101-Md-259



$\times 10^{-6}$

Legendre (CM)



30

25

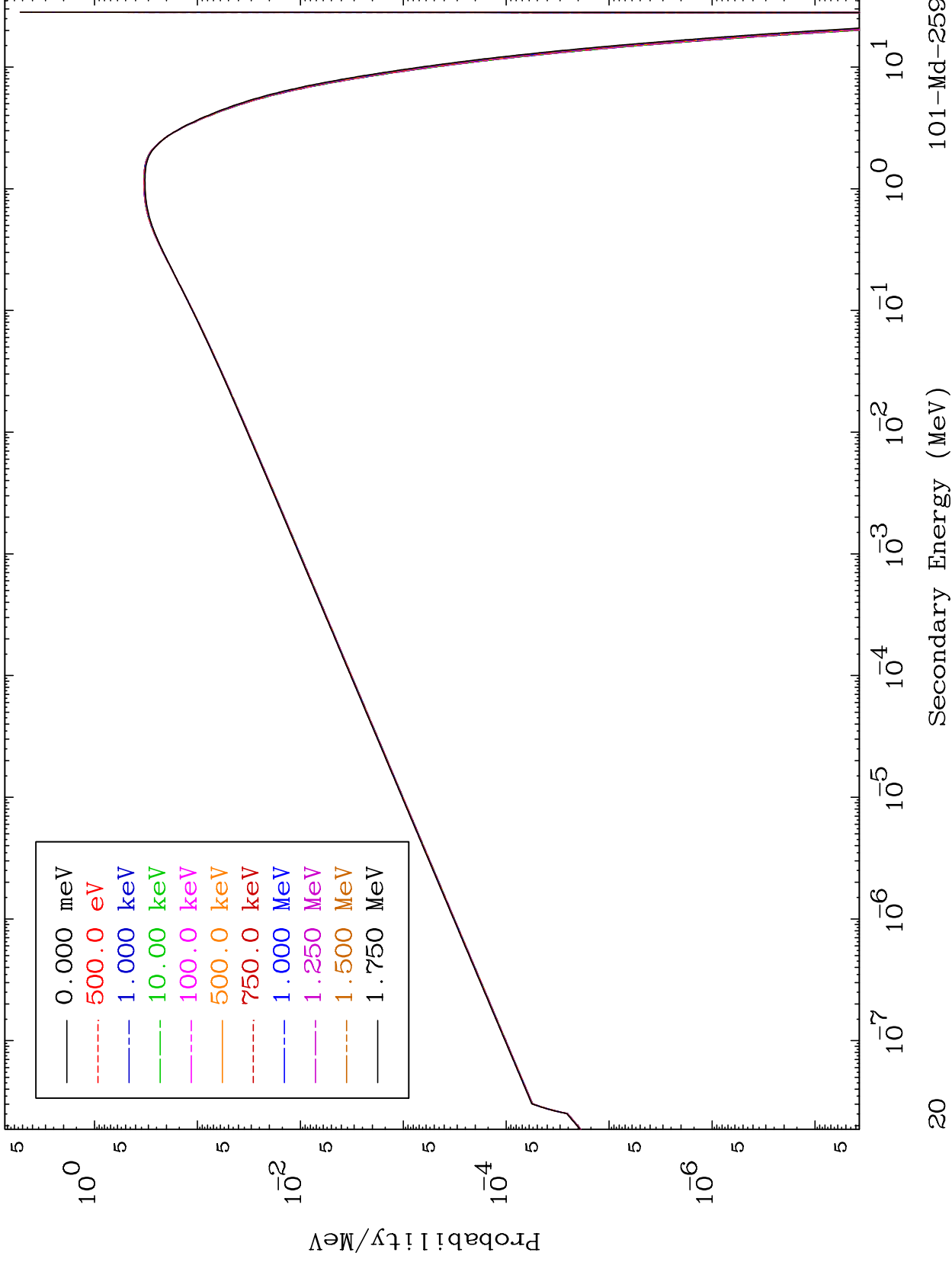
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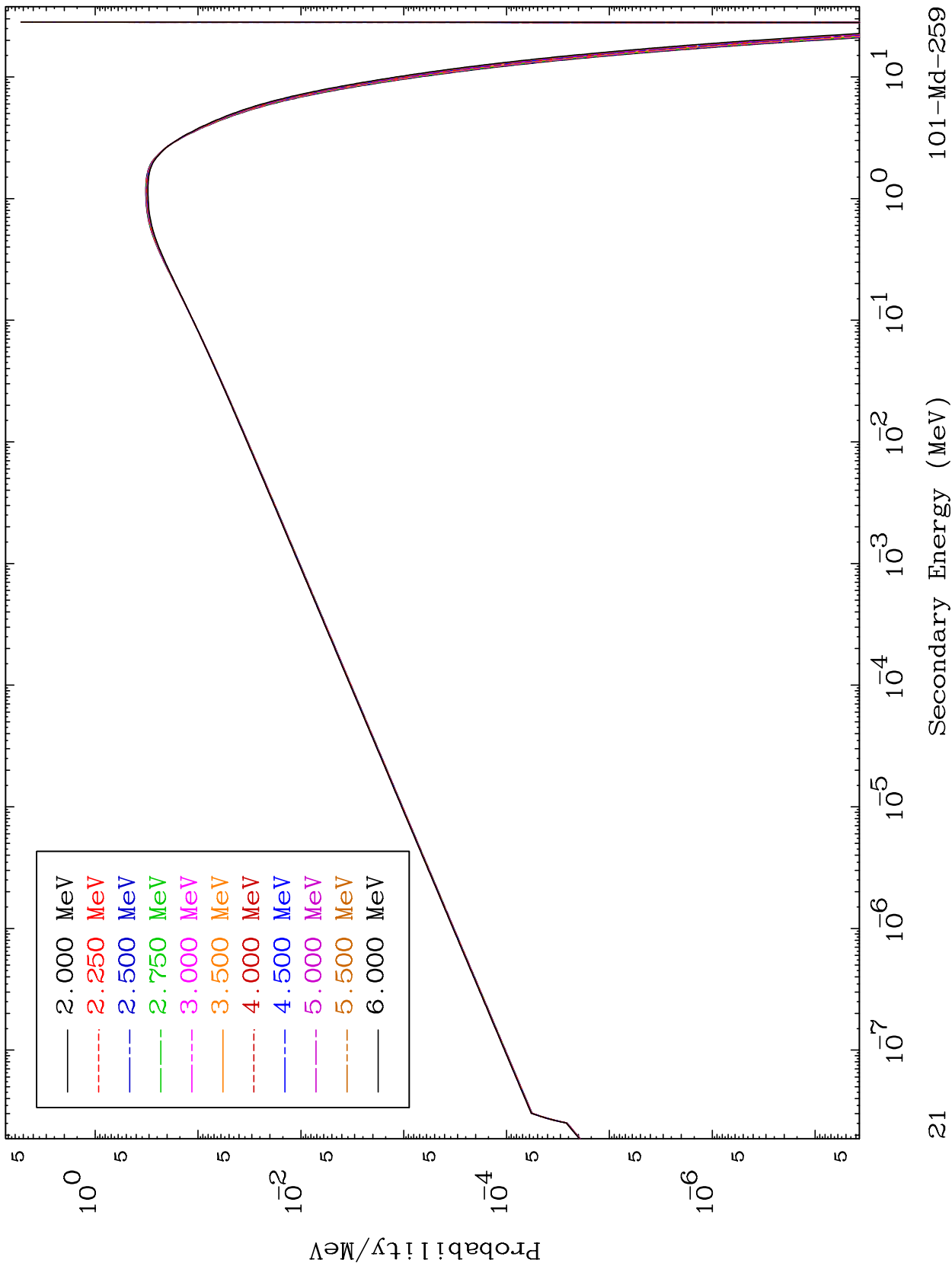
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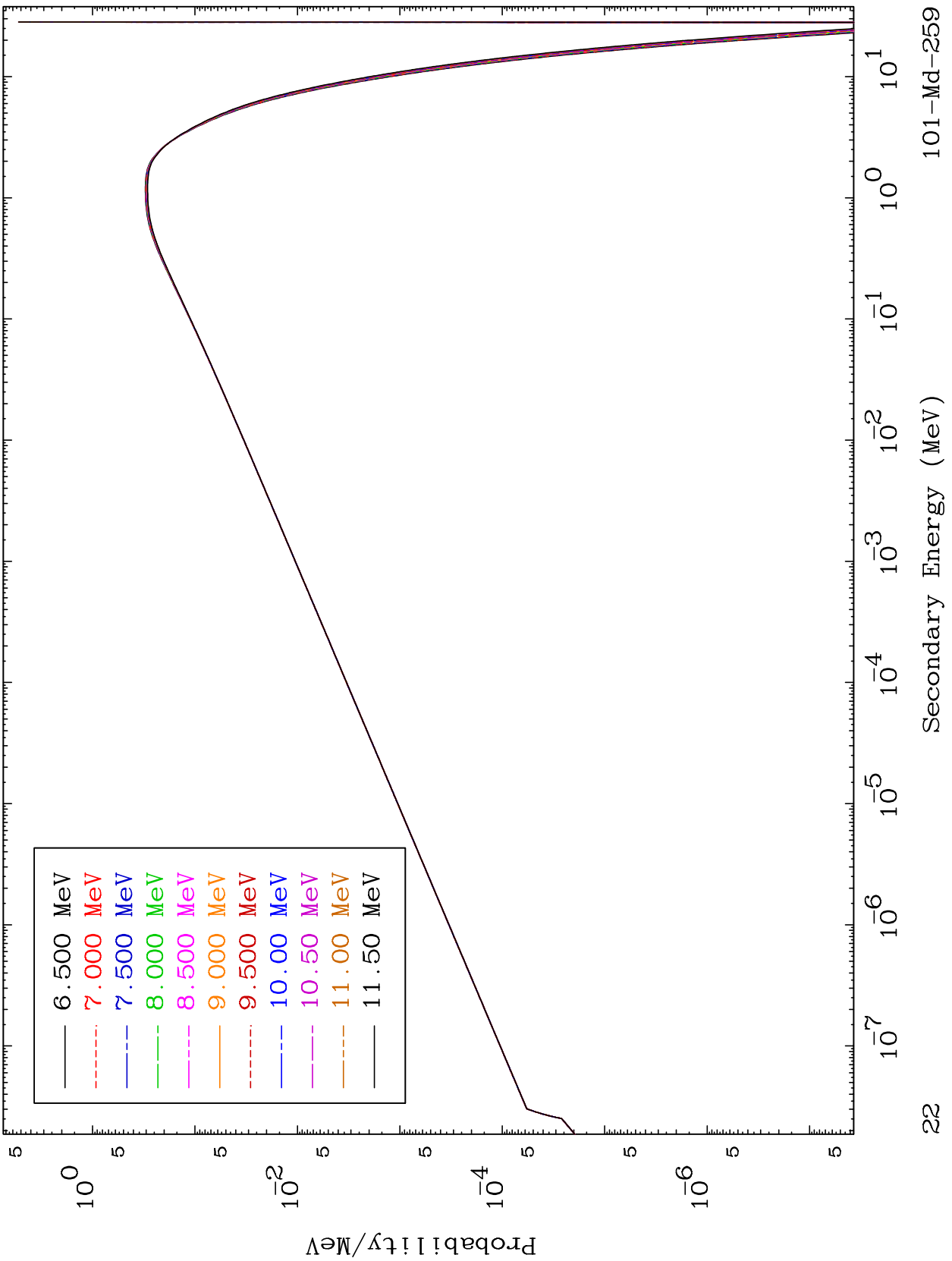
19

Incident Energy (MeV)

101-Md-259



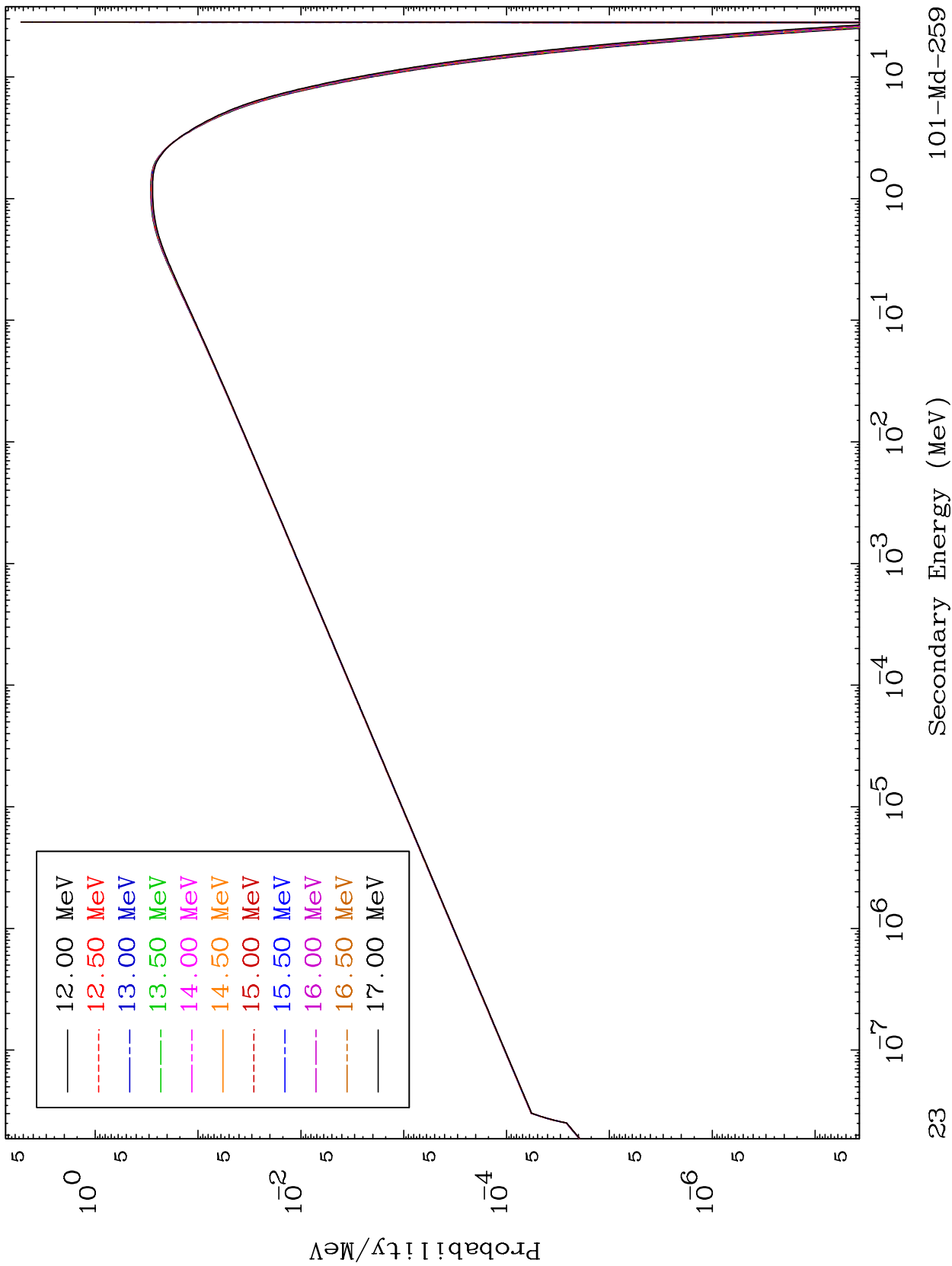




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Fission Energy Distribution

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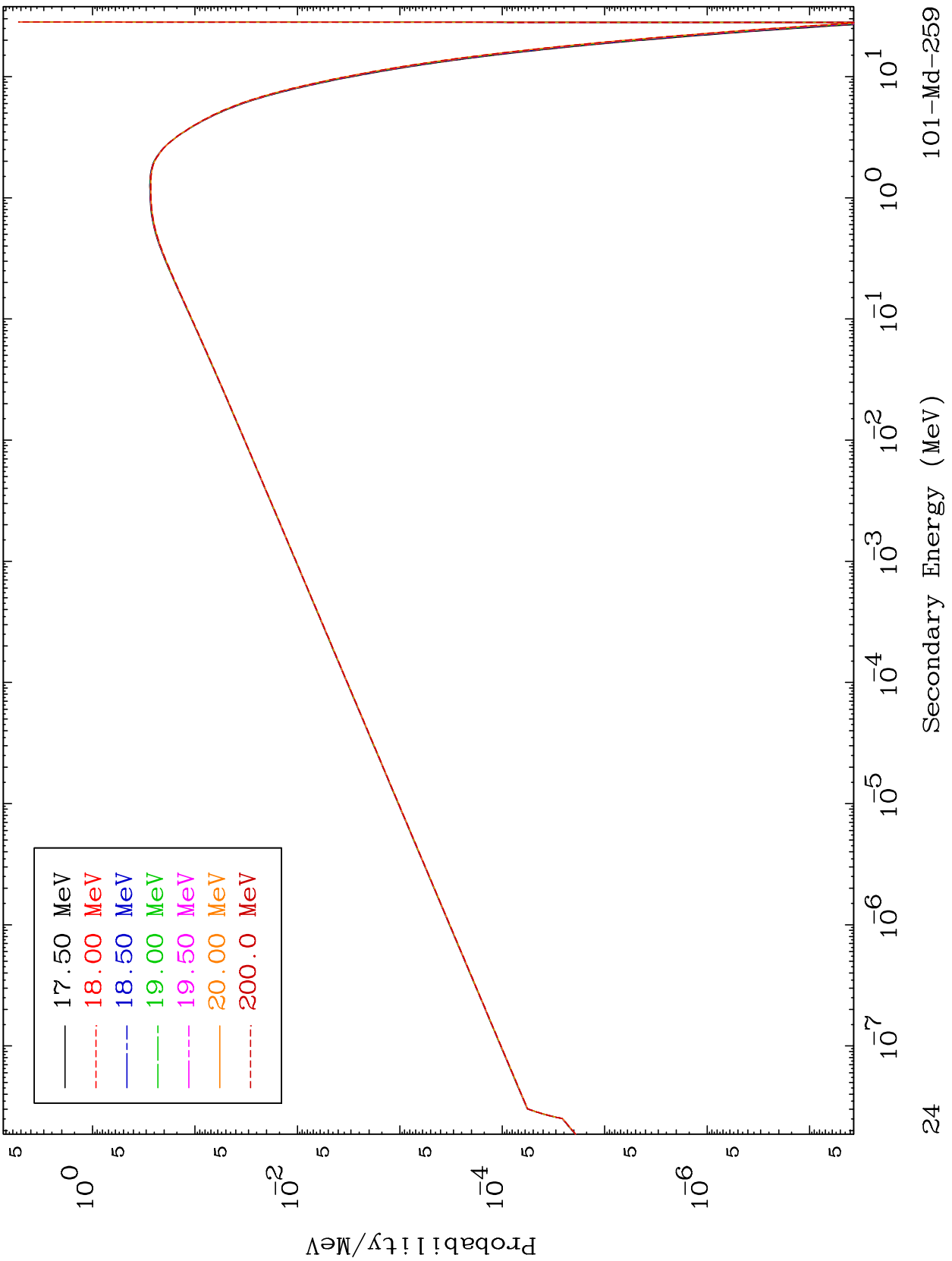


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Fission Energy Distribution

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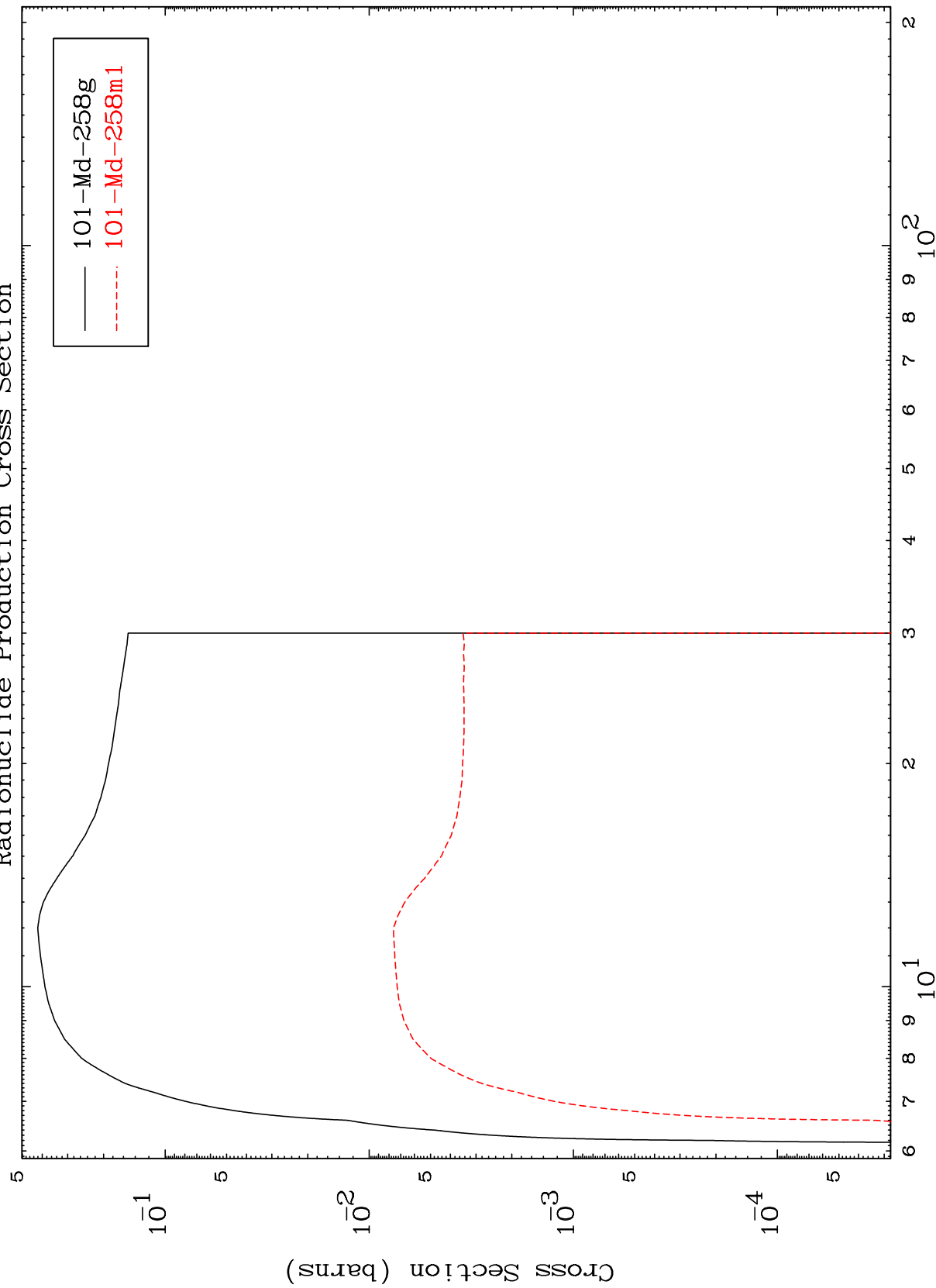


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(n,2n)

Radionuclide Production Cross Section



Incident Energy (MeV)

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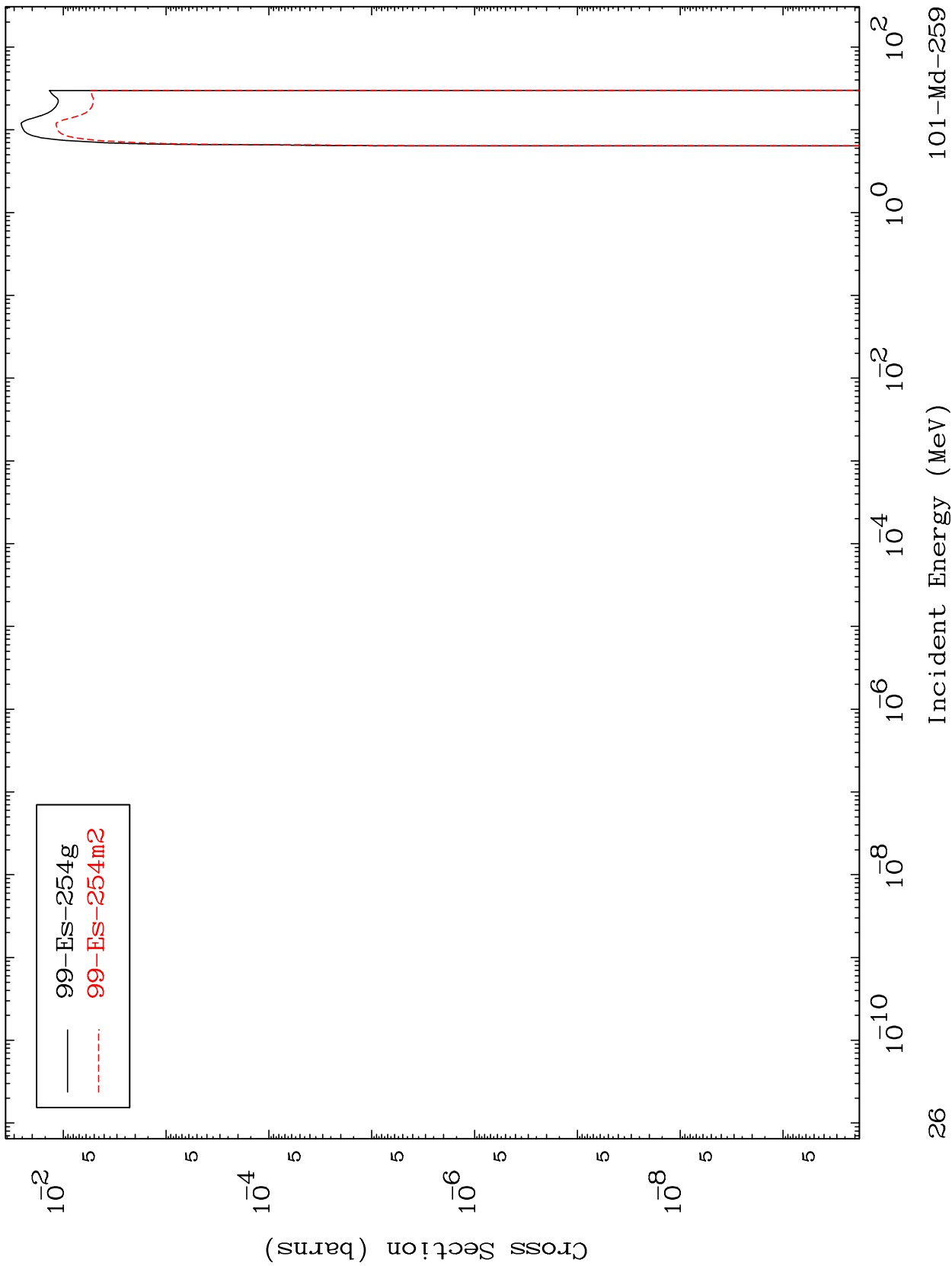
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(n,2n) α

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Radionuclide Production Cross Section

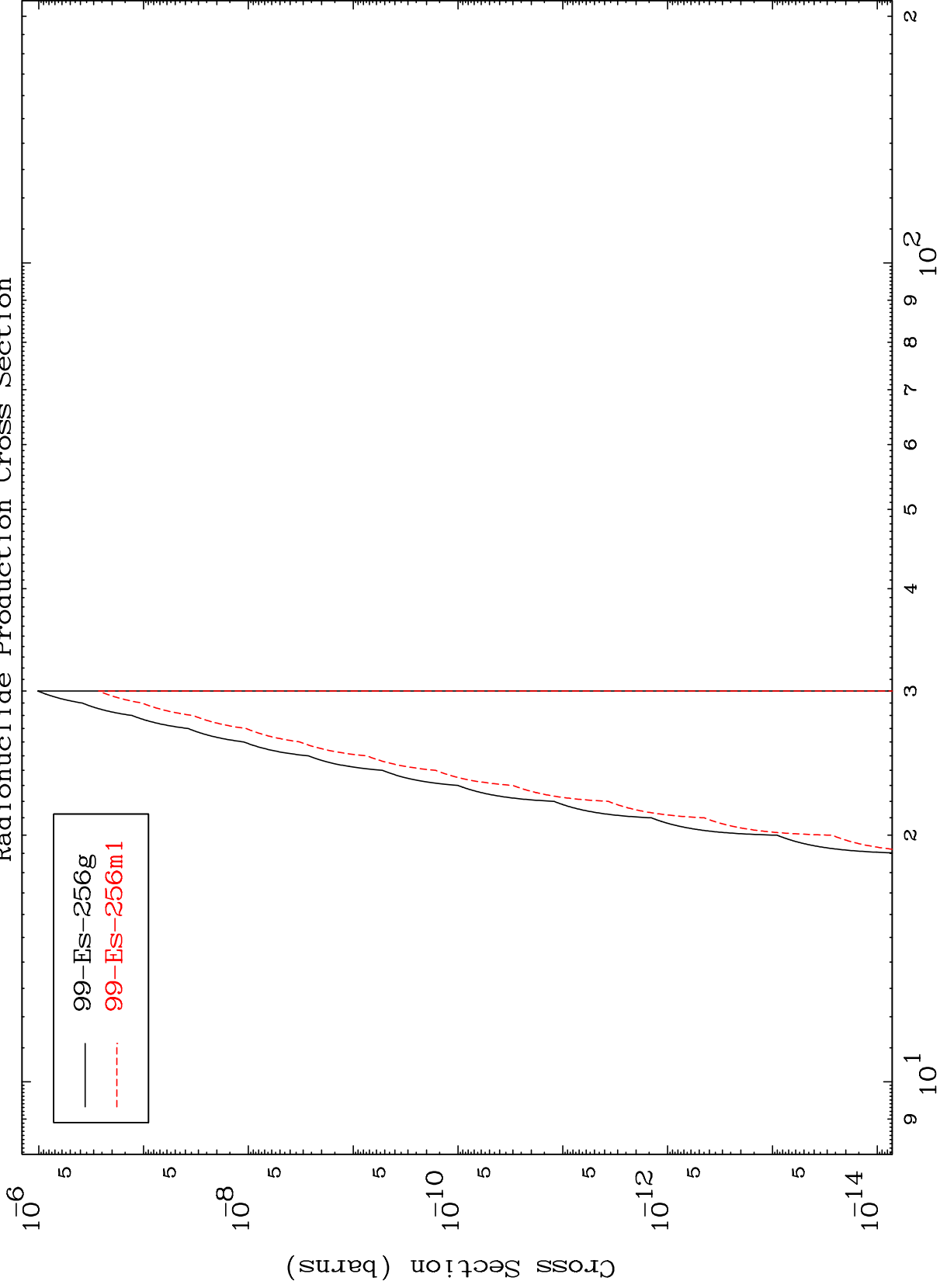


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(n,n') He-3

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Radionuclide Production Cross Section



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Incident Energy (MeV)

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