

Program EVALPLOT
(Version 2015-2)

by

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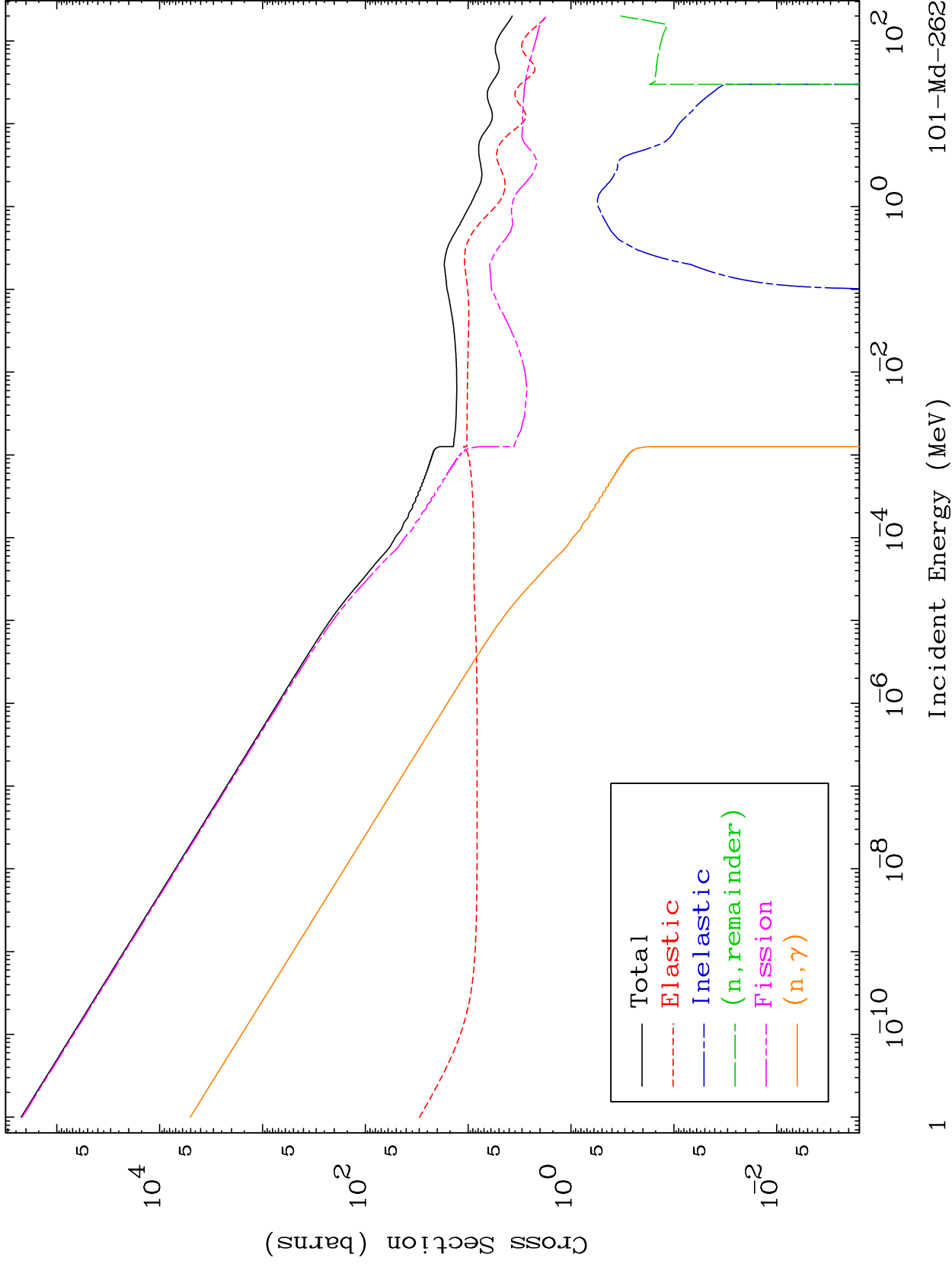
Press Mouse Button to Start

MAT 9963

Major

293 Kelvin Cross Sections

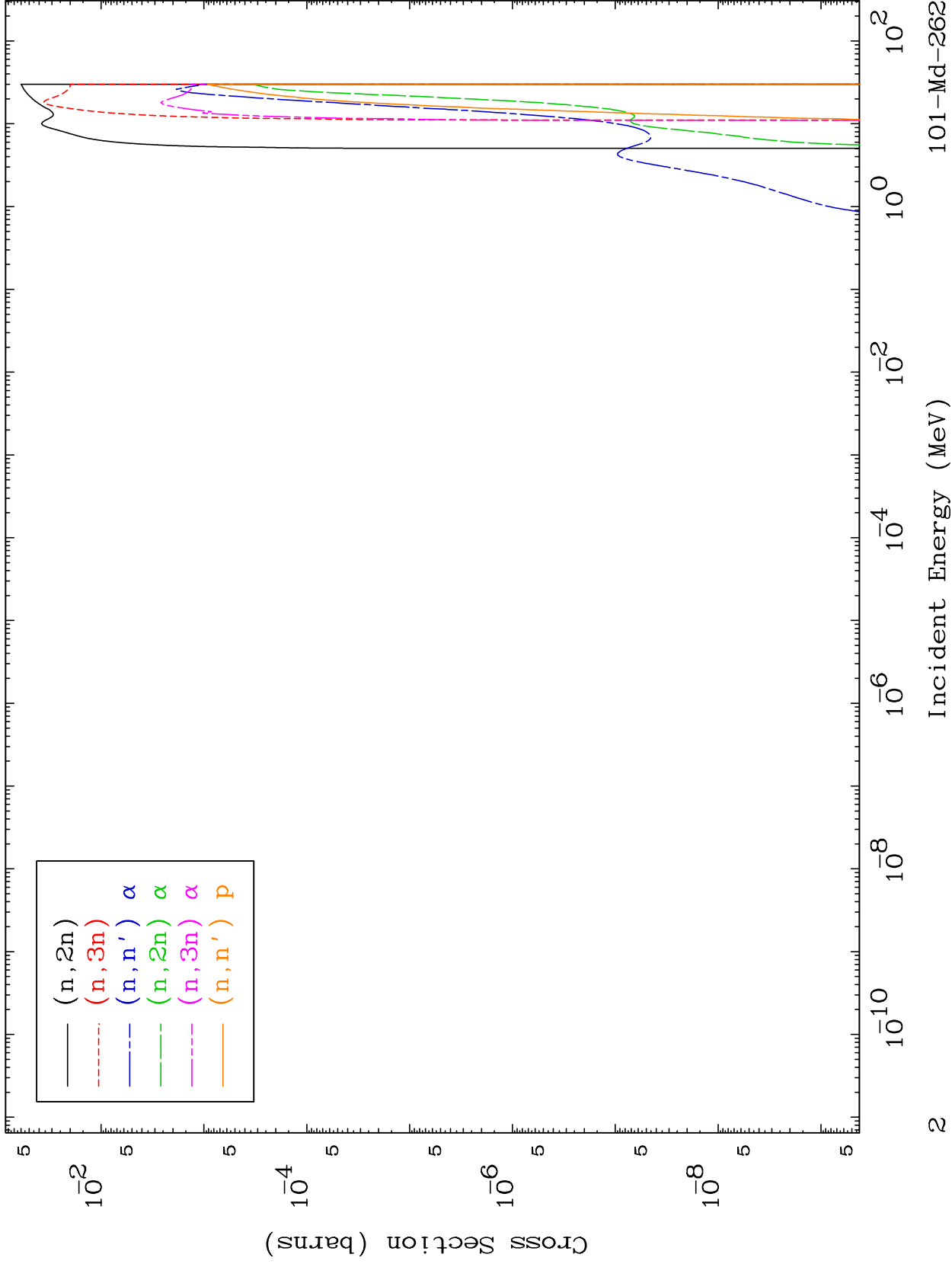
101-Md-262

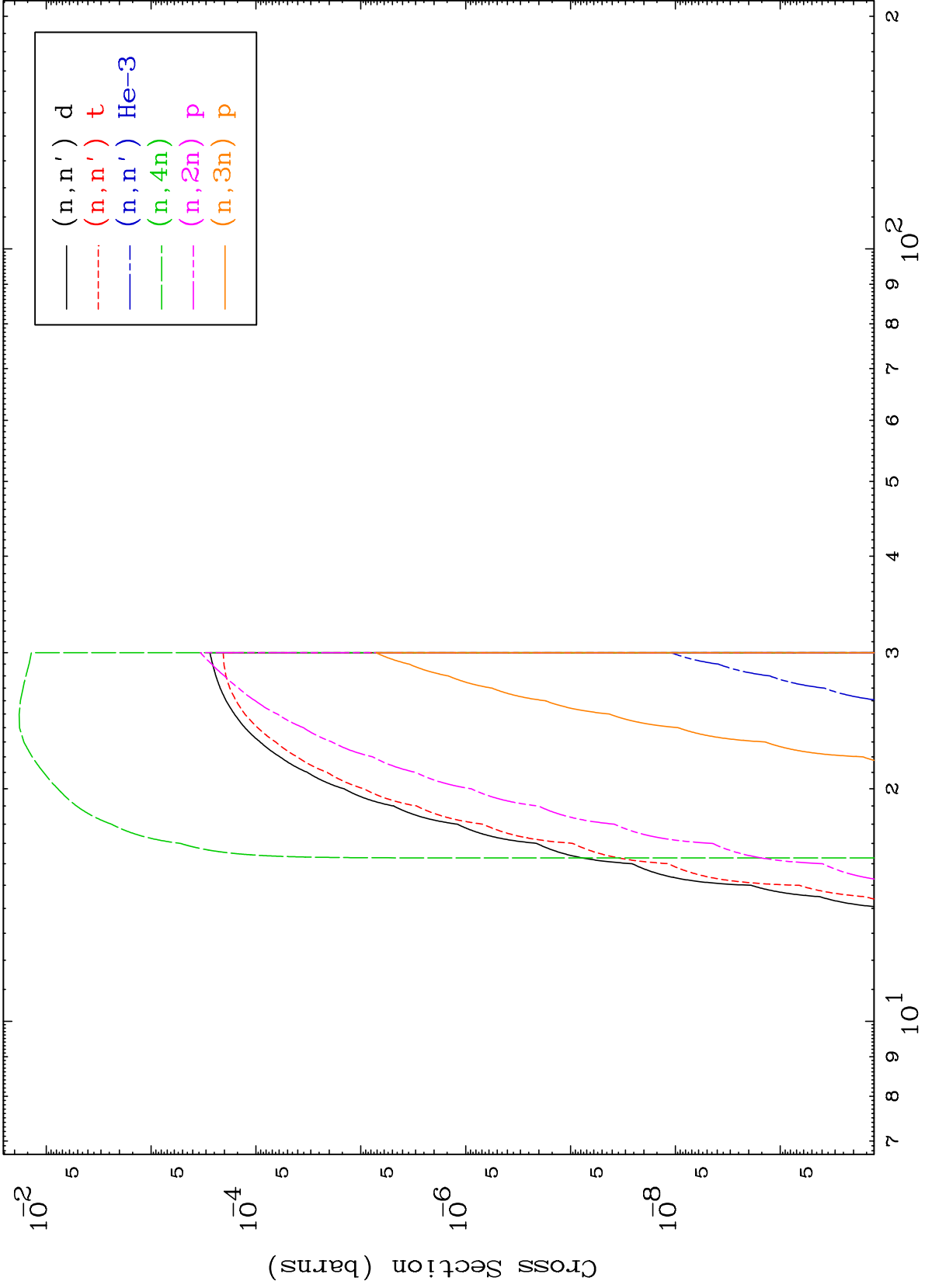


MAT 9963

Neutron Production
293 Kelvin Cross Sections

101-Md-262

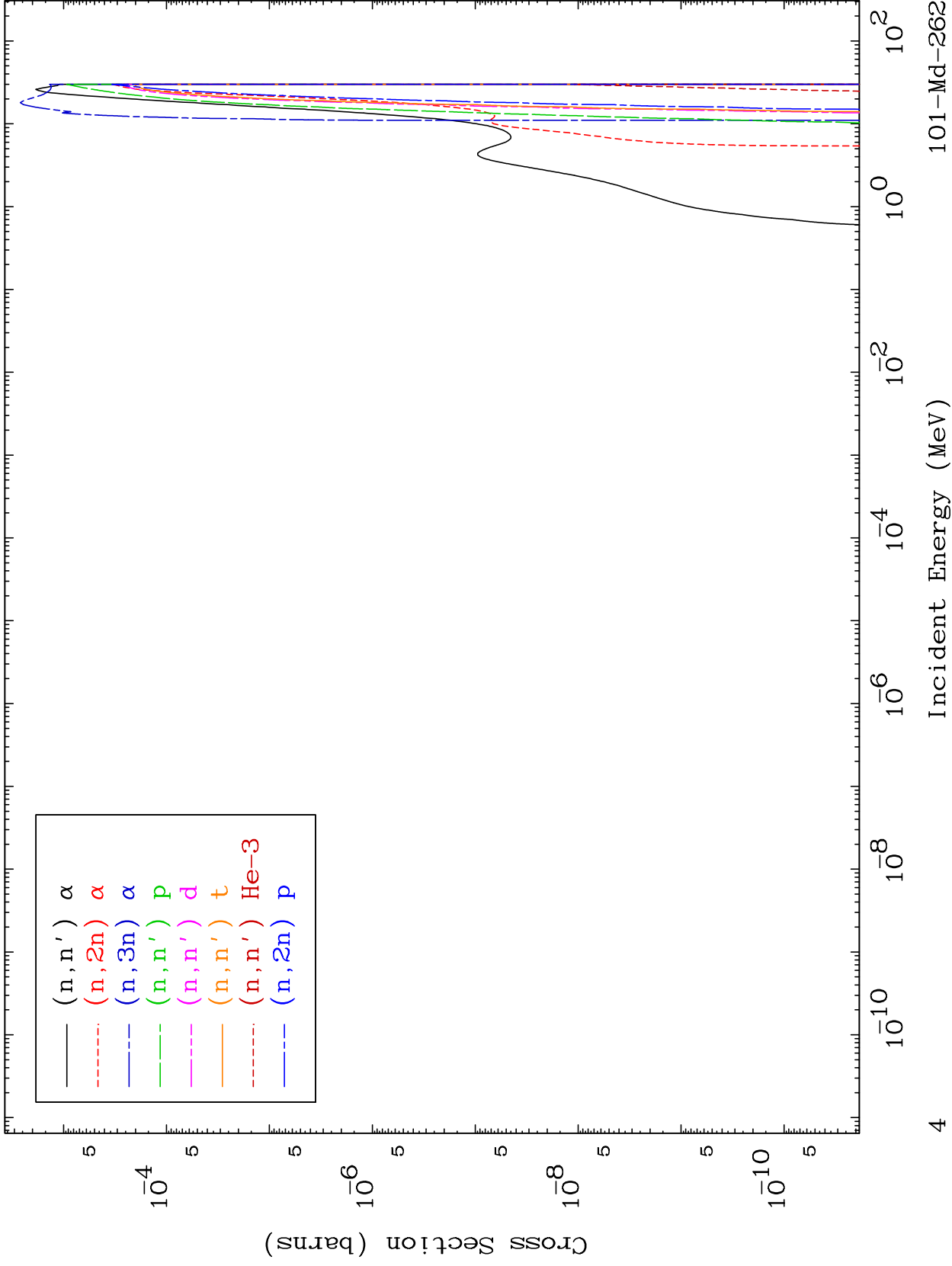




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Charged Particle
293 Kelvin Cross Sections

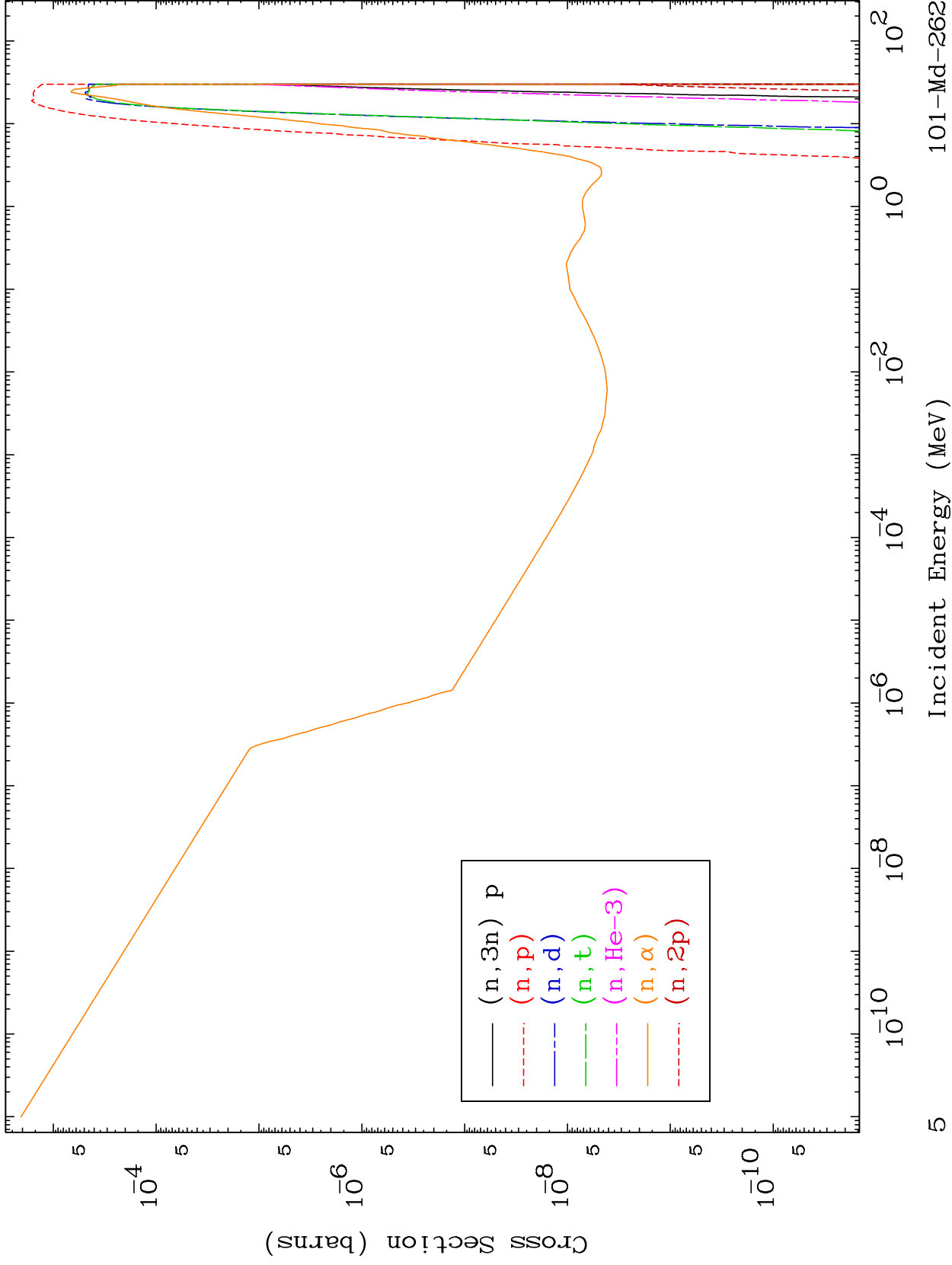
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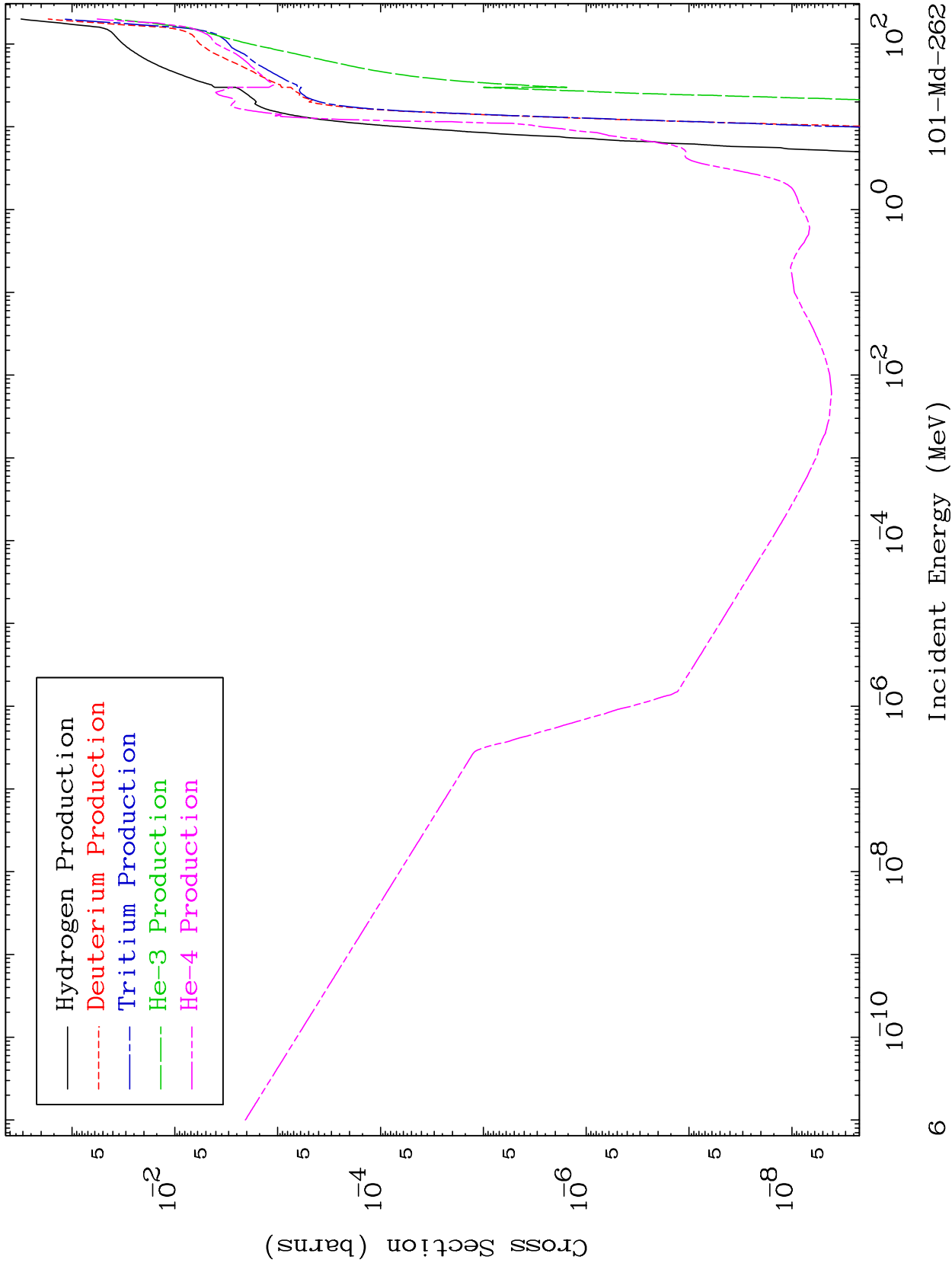


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Charged Particle
293 Kelvin Cross Sections

101-Md-262

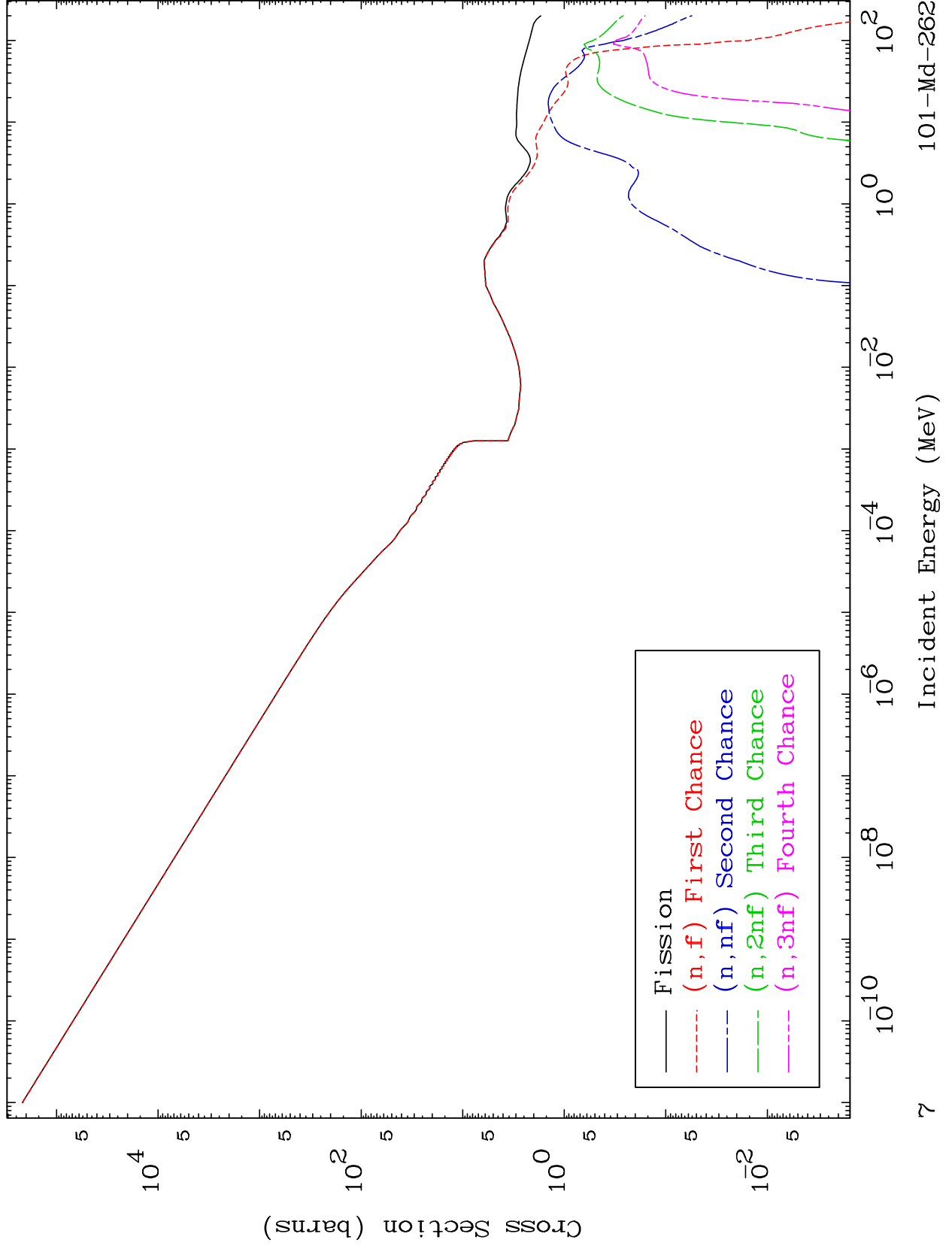




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Fission
293 Kelvin Cross Sections

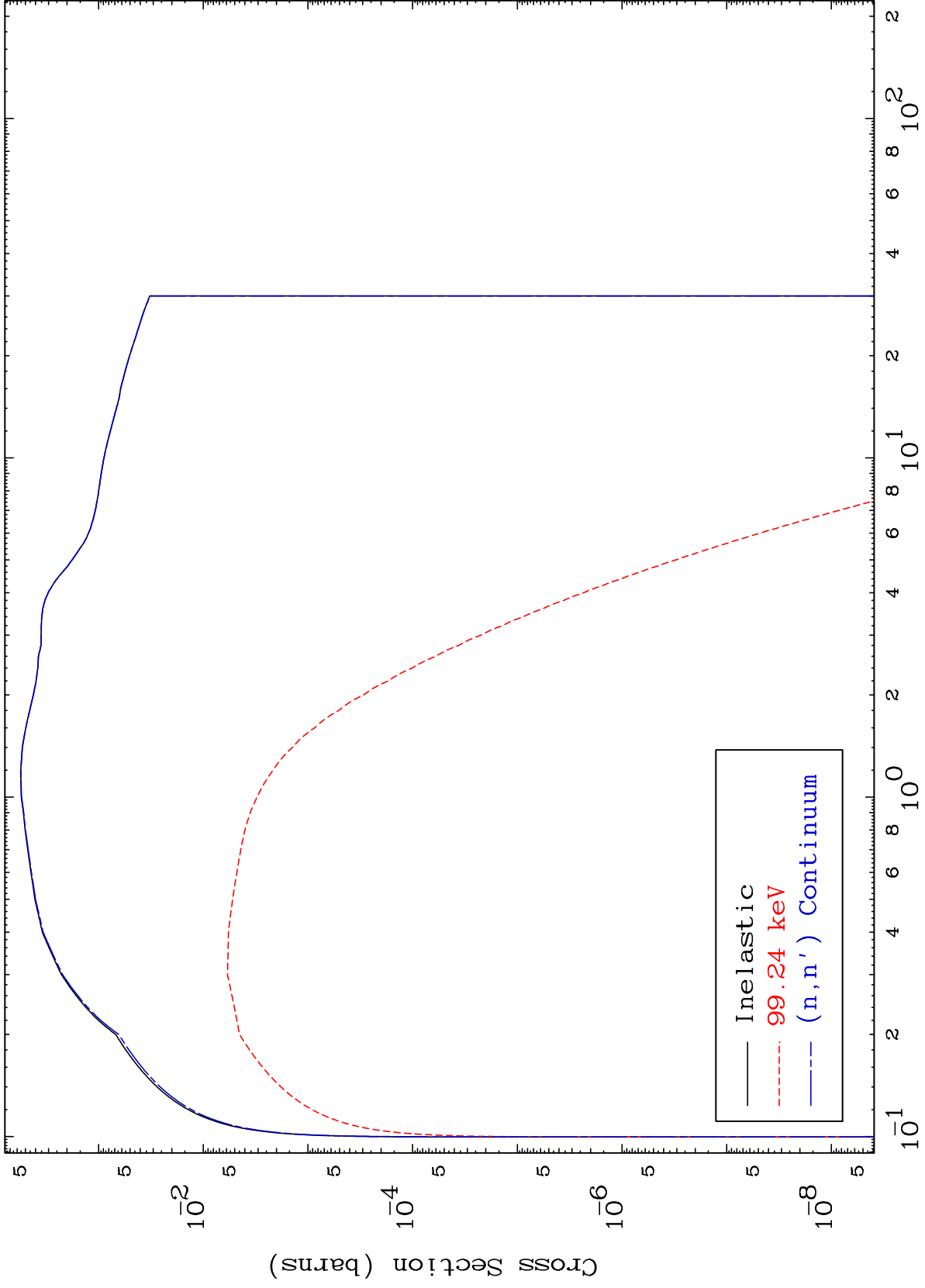
101-Md-262



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(n,n') Level
293 Kelvin Cross Sections

101-Md-262



Incident Energy (MeV)

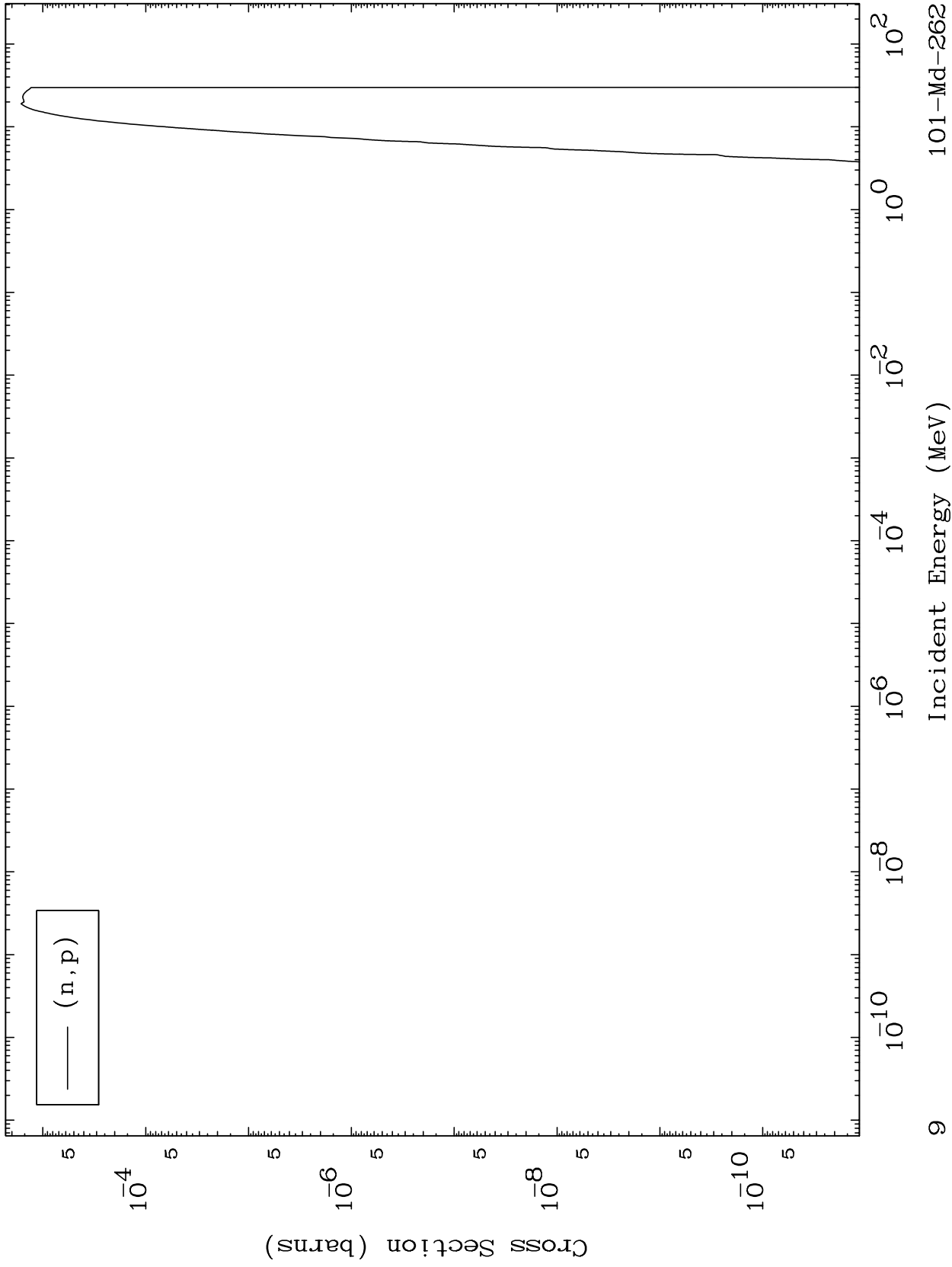
101-Md-262

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(n,p) Levels
293 Kelvin Cross Sections

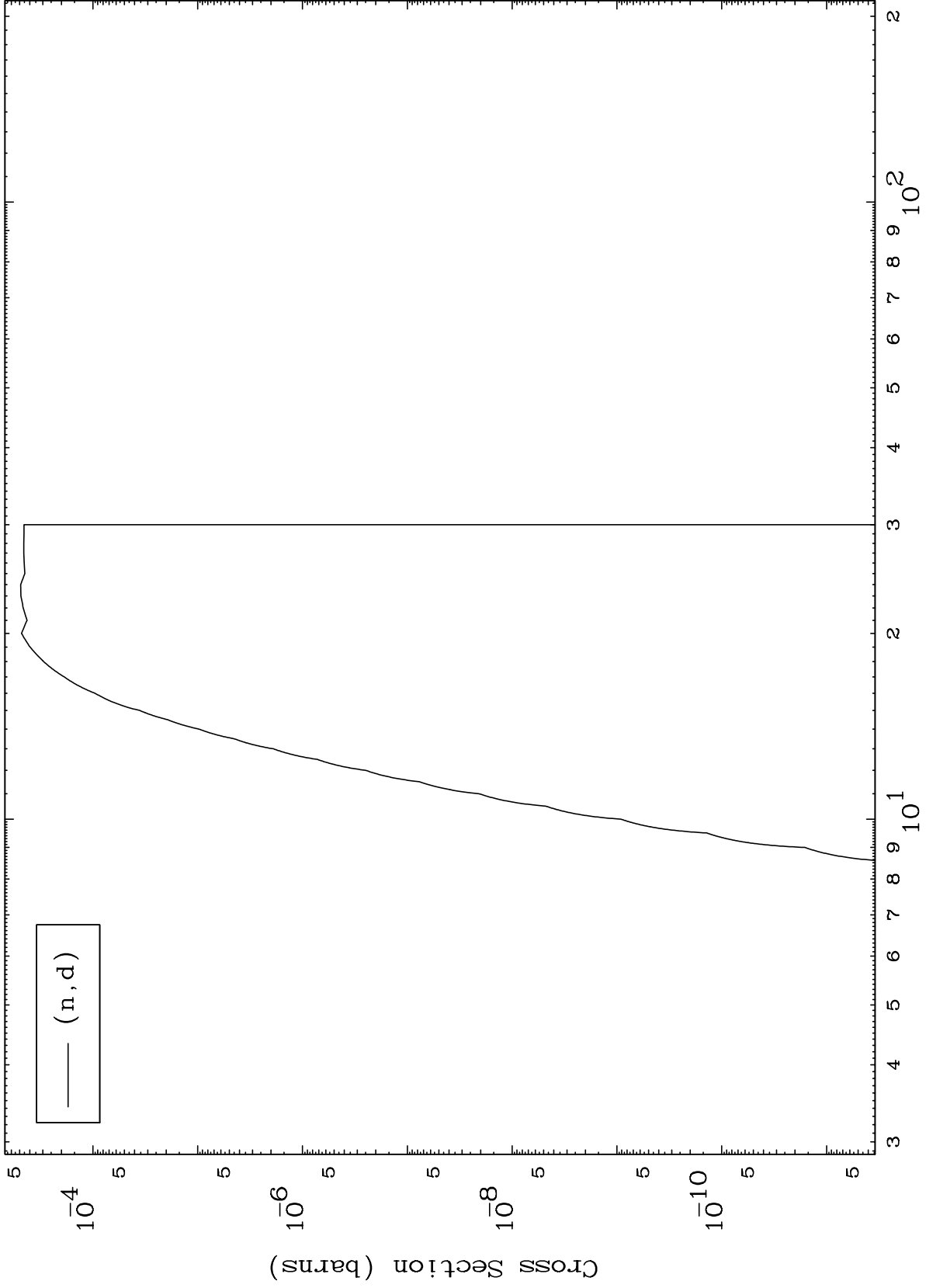
101-Md-262



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(n,d) Levels
293 Kelvin Cross Sections

101-Md-262



10

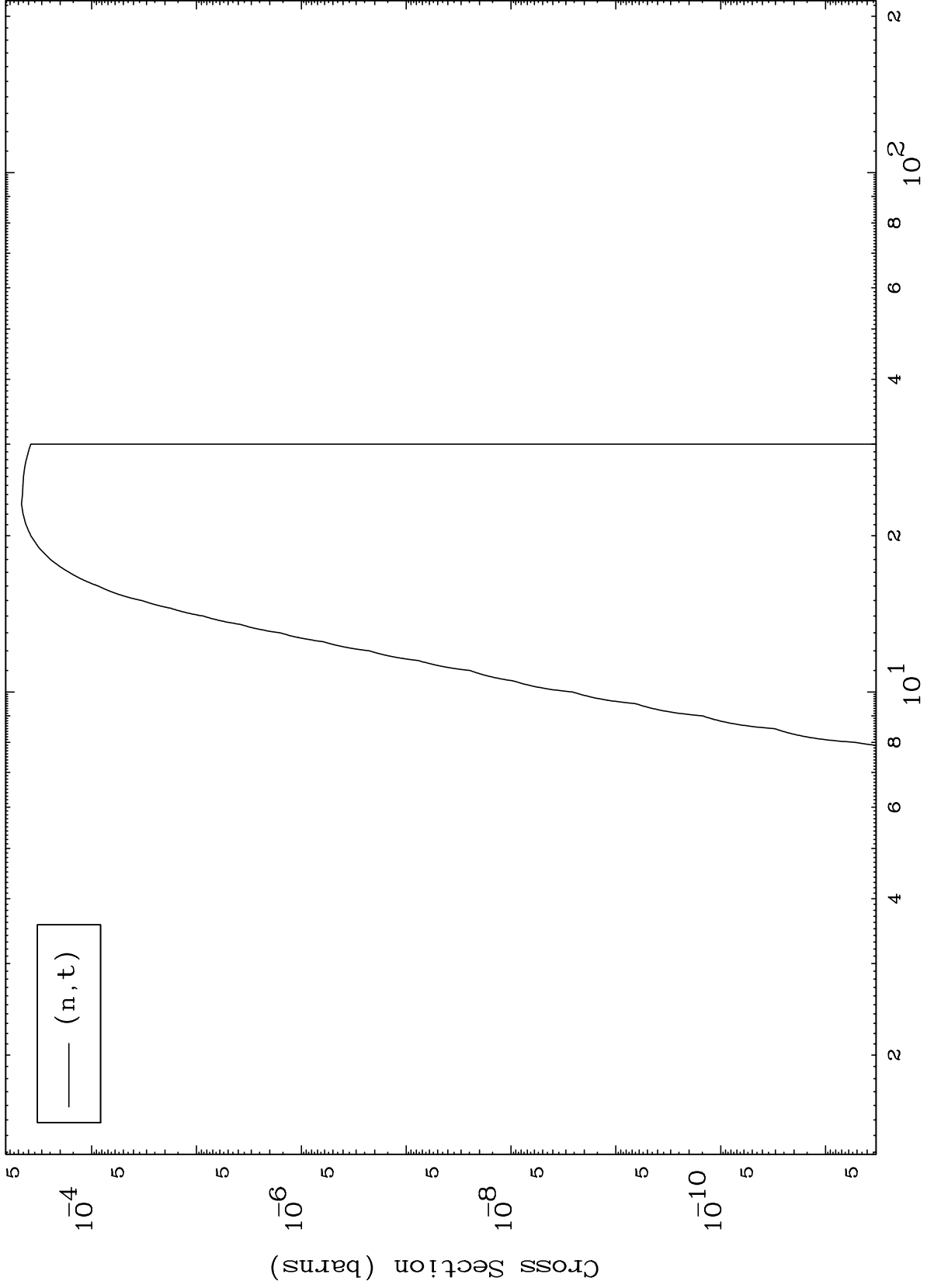
Incident Energy (MeV)

101-Md-262

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(n,t) Levels
293 Kelvin Cross Sections

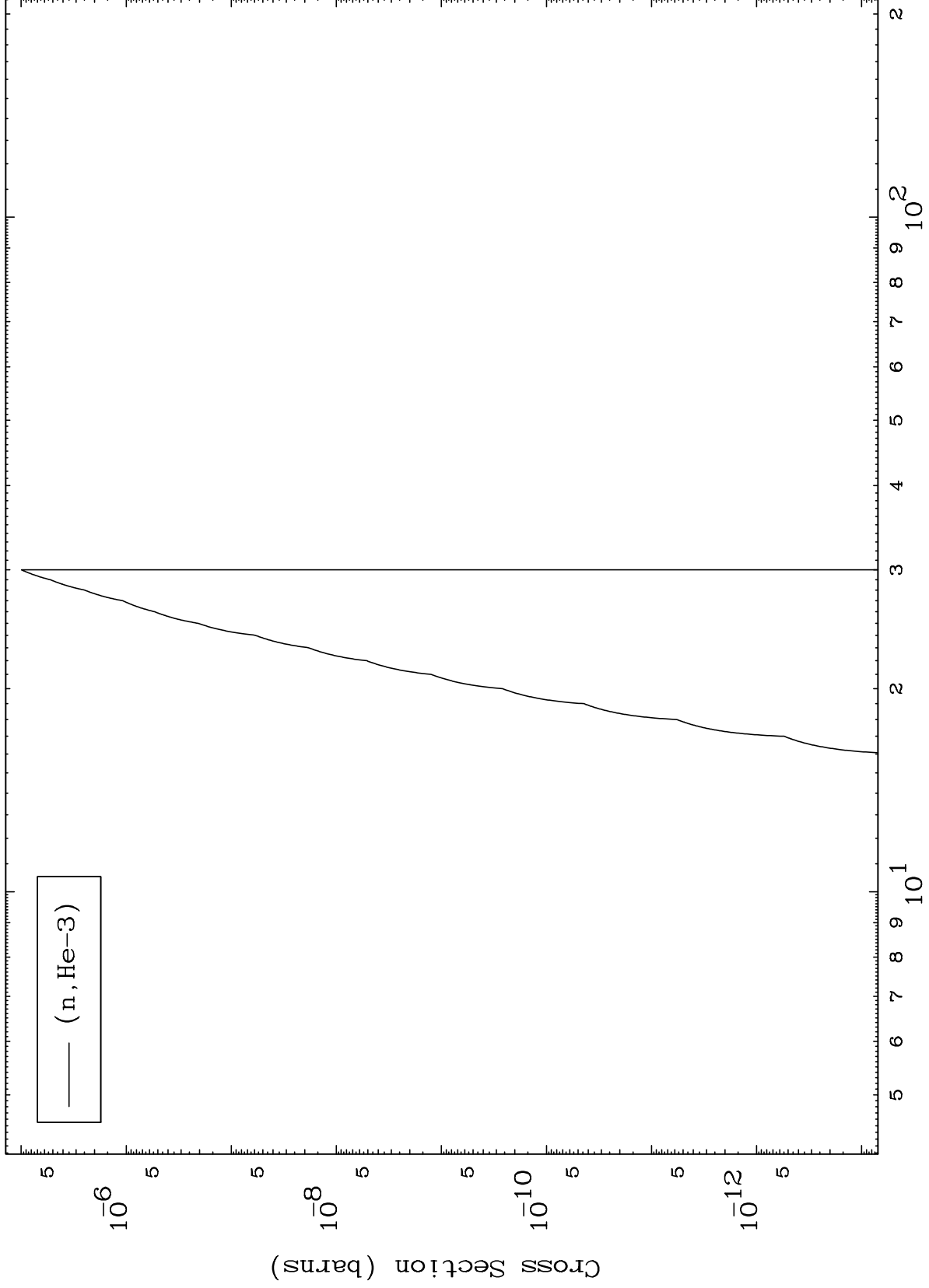
101-Md-262



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(n,He3) Levels
293 Kelvin Cross Sections

101-Md-262



12

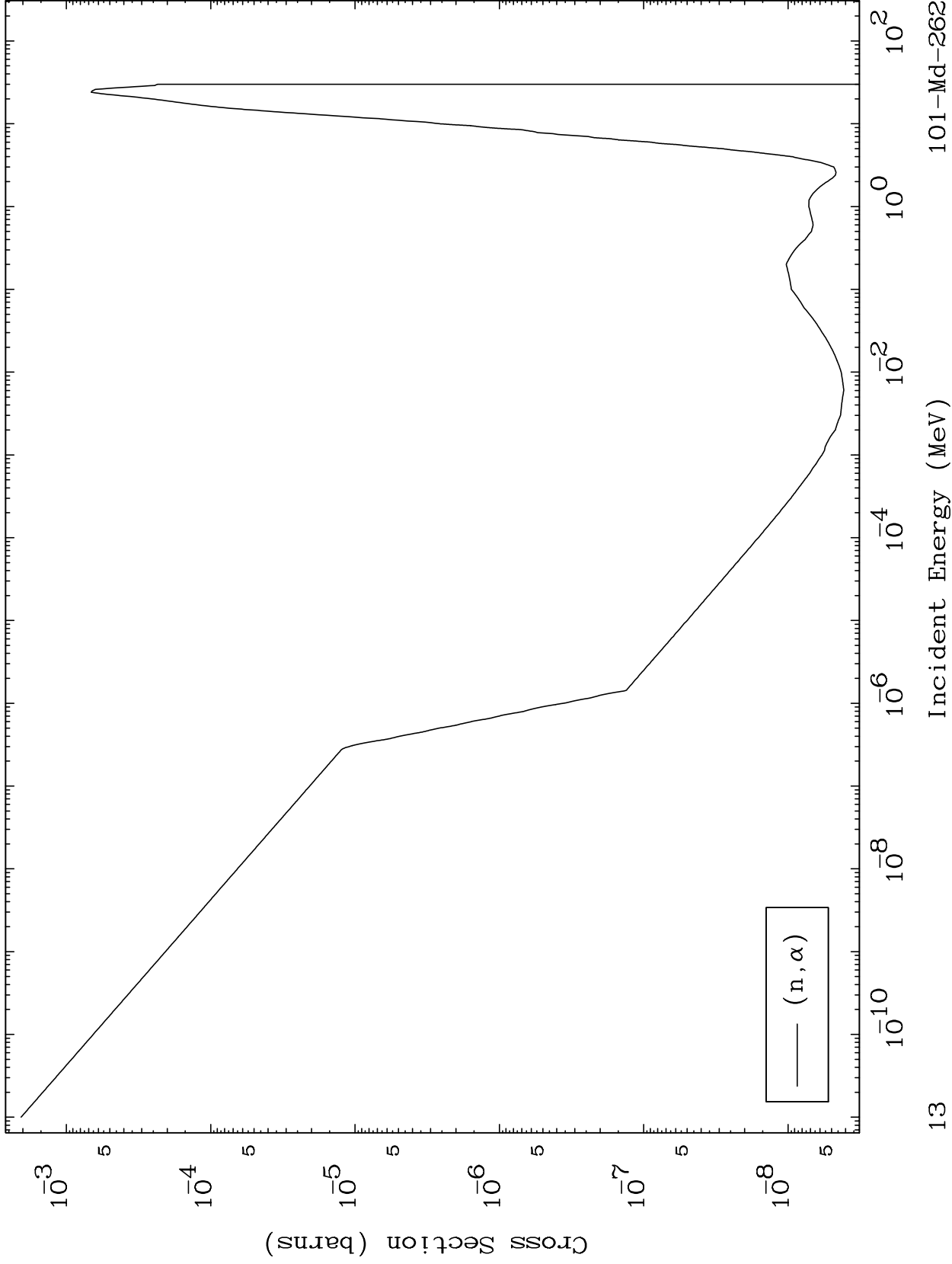
Incident Energy (MeV)

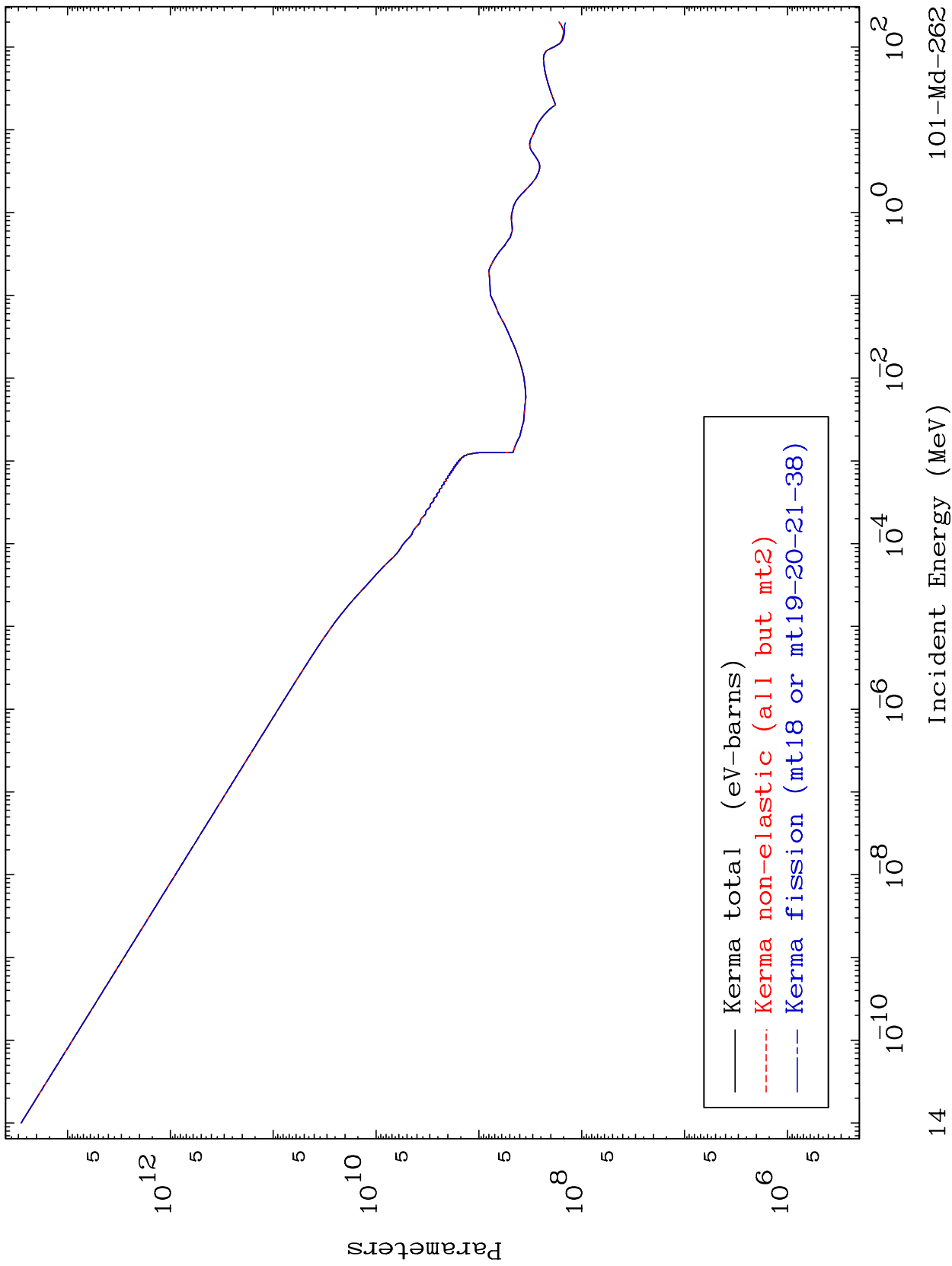
101-Md-262

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(n, α) Levels
293 Kelvin Cross Sections

101-Md-262

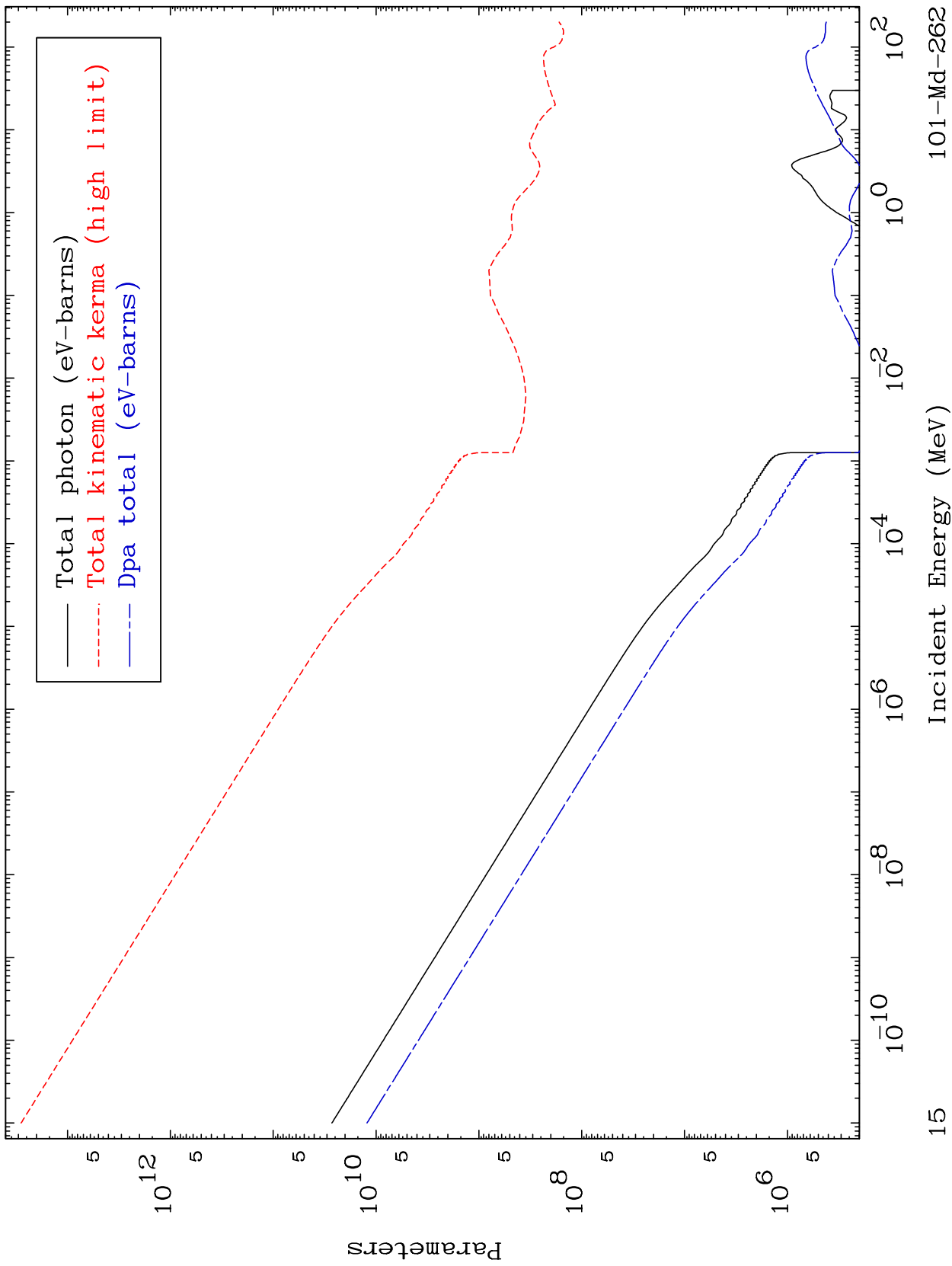


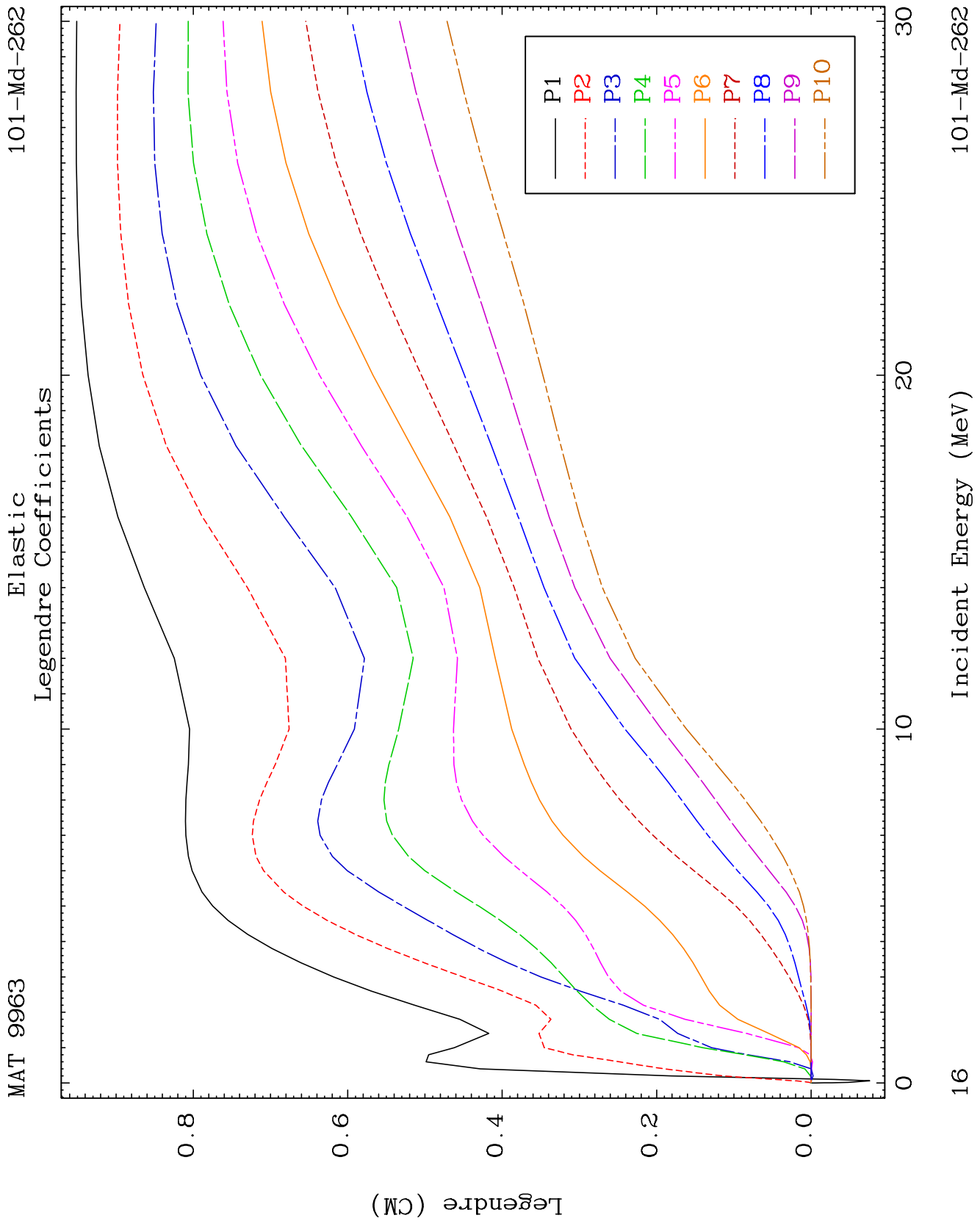


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Energy Release
Parameters

101-Md-262

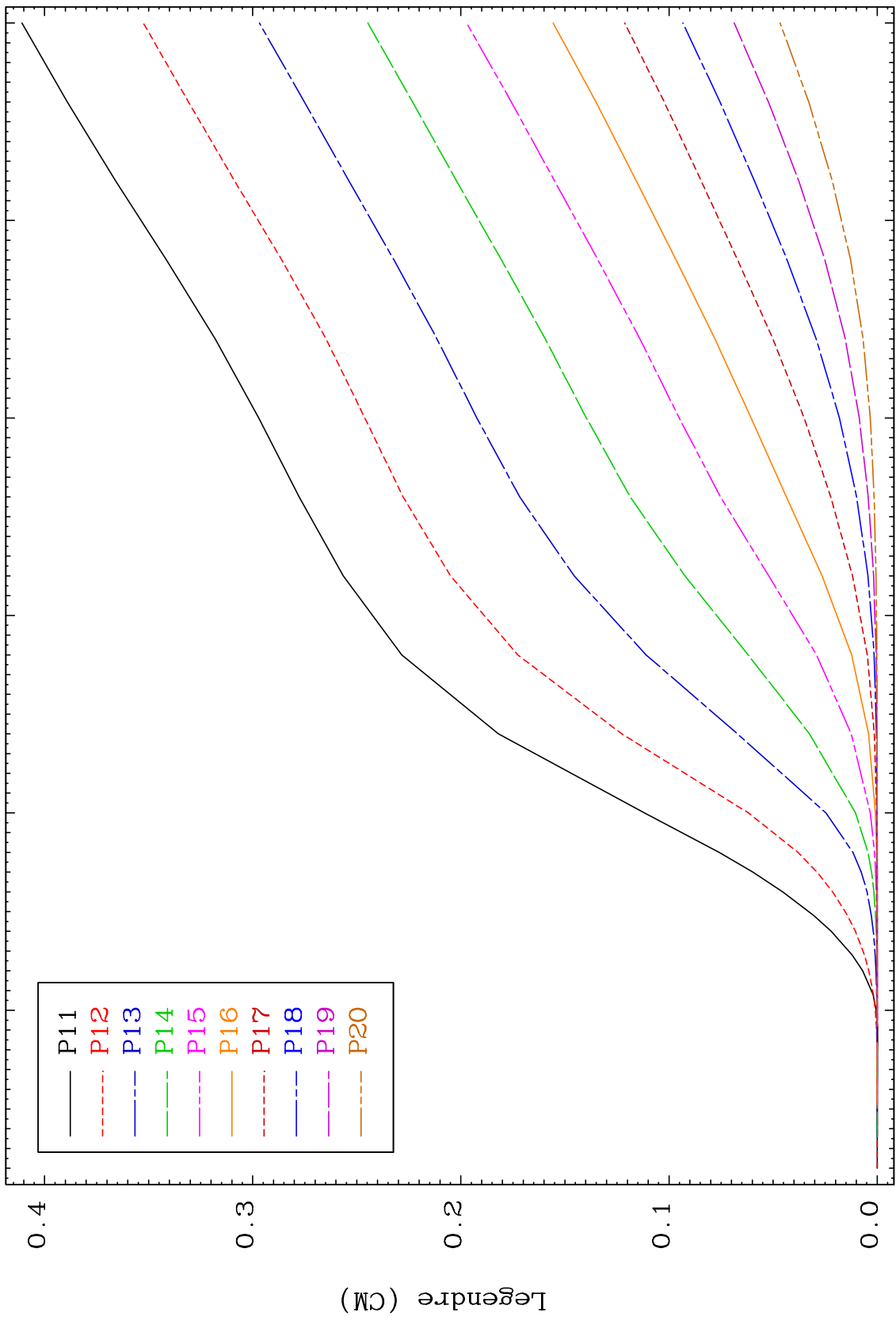




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Elastic Legendre Coefficients

101-Md-262



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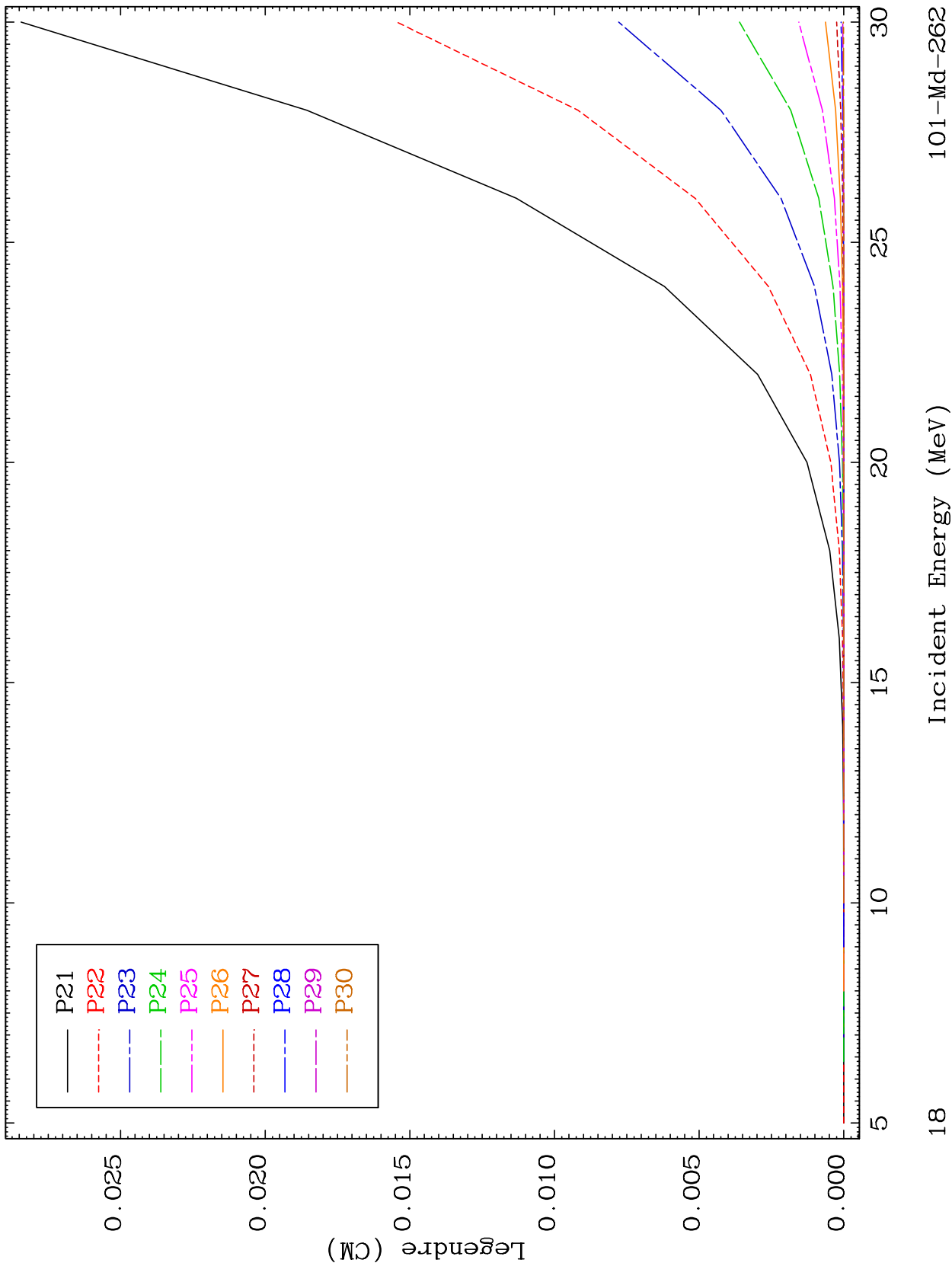
Incident Energy (MeV)

101-Md-262

MAT 9963

Elastic Legendre Coefficients

101-Md-262



18

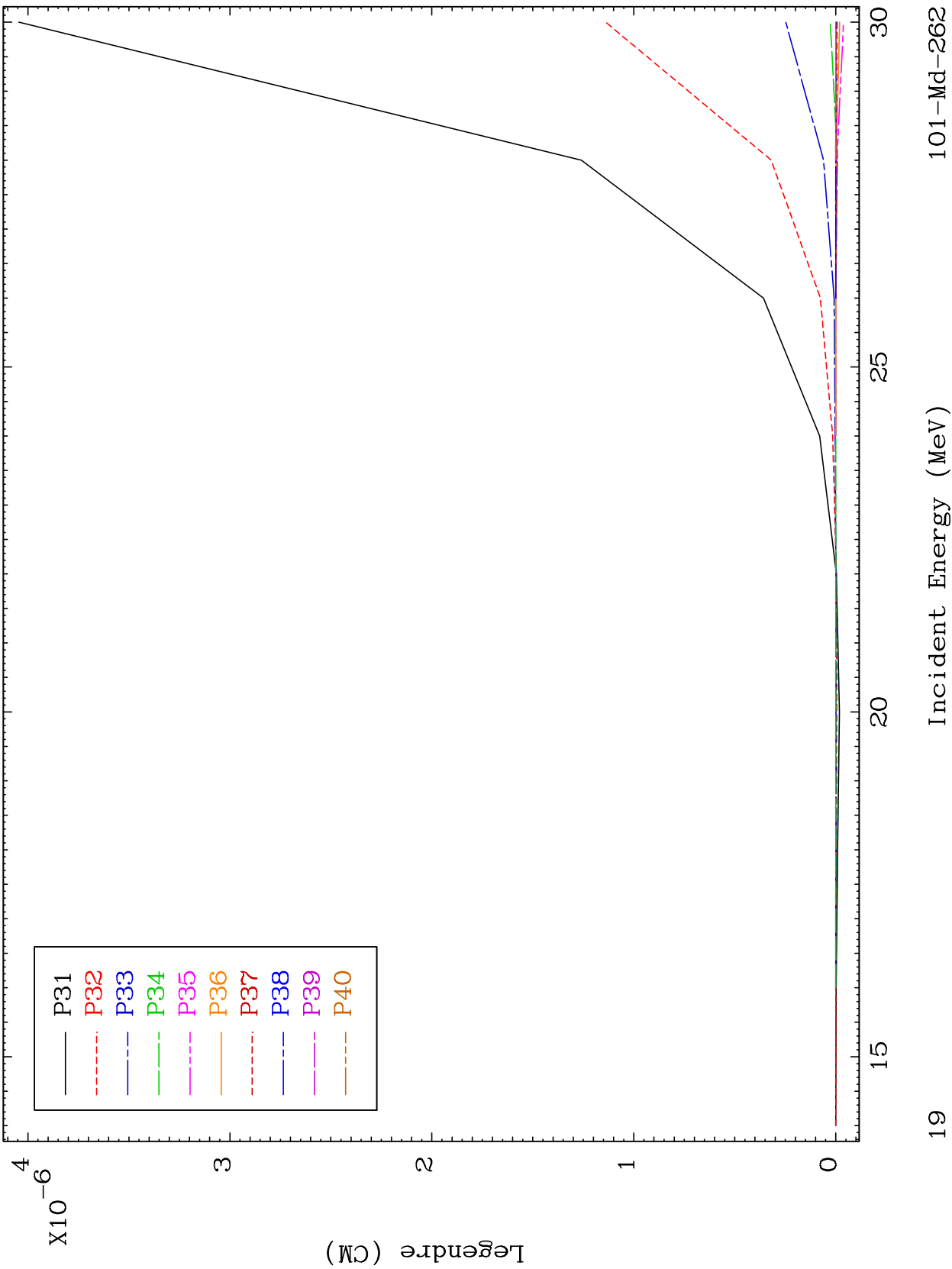
Incident Energy (MeV)

101-Md-262

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Elastic
Legendre Coefficients

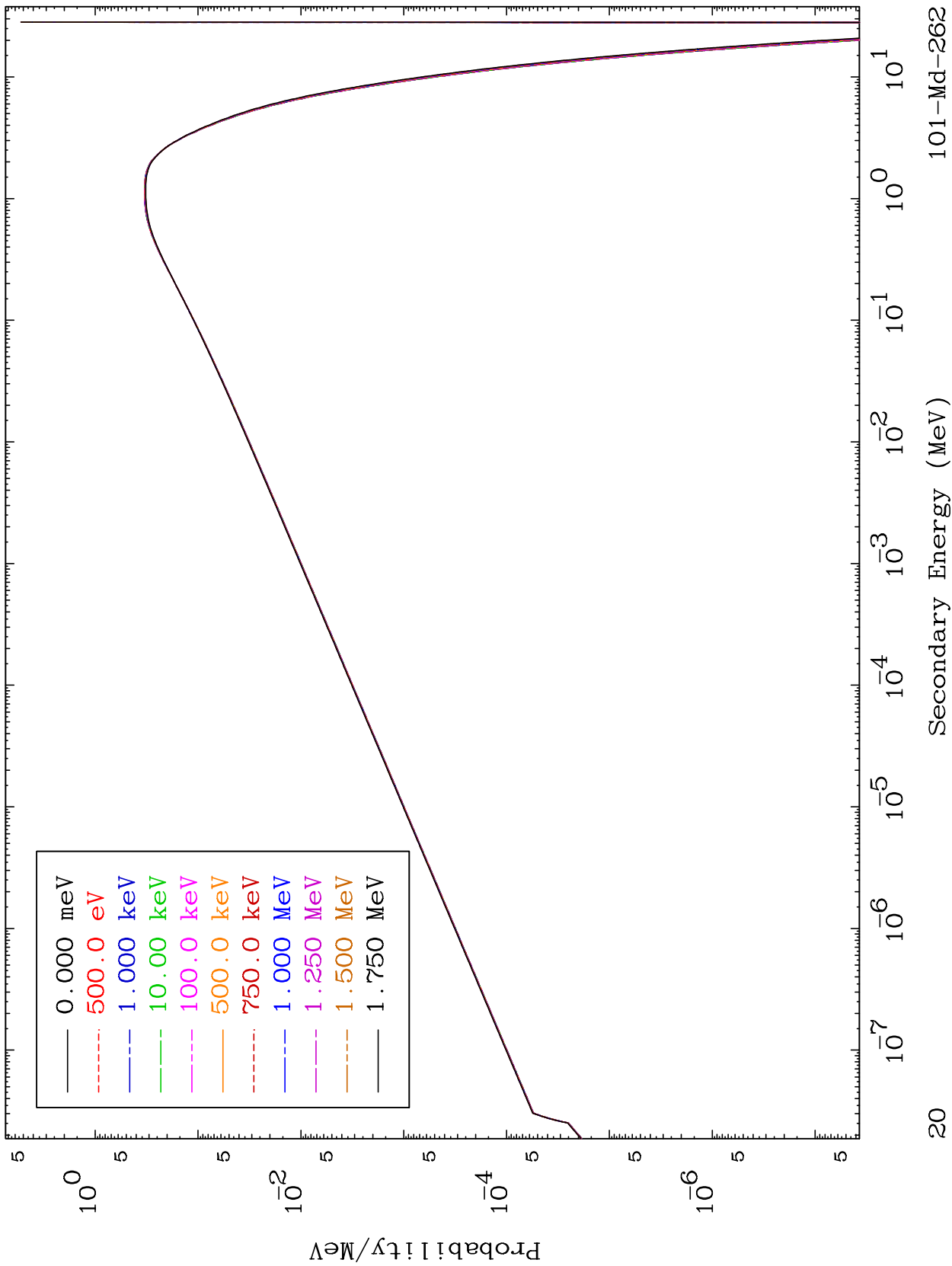
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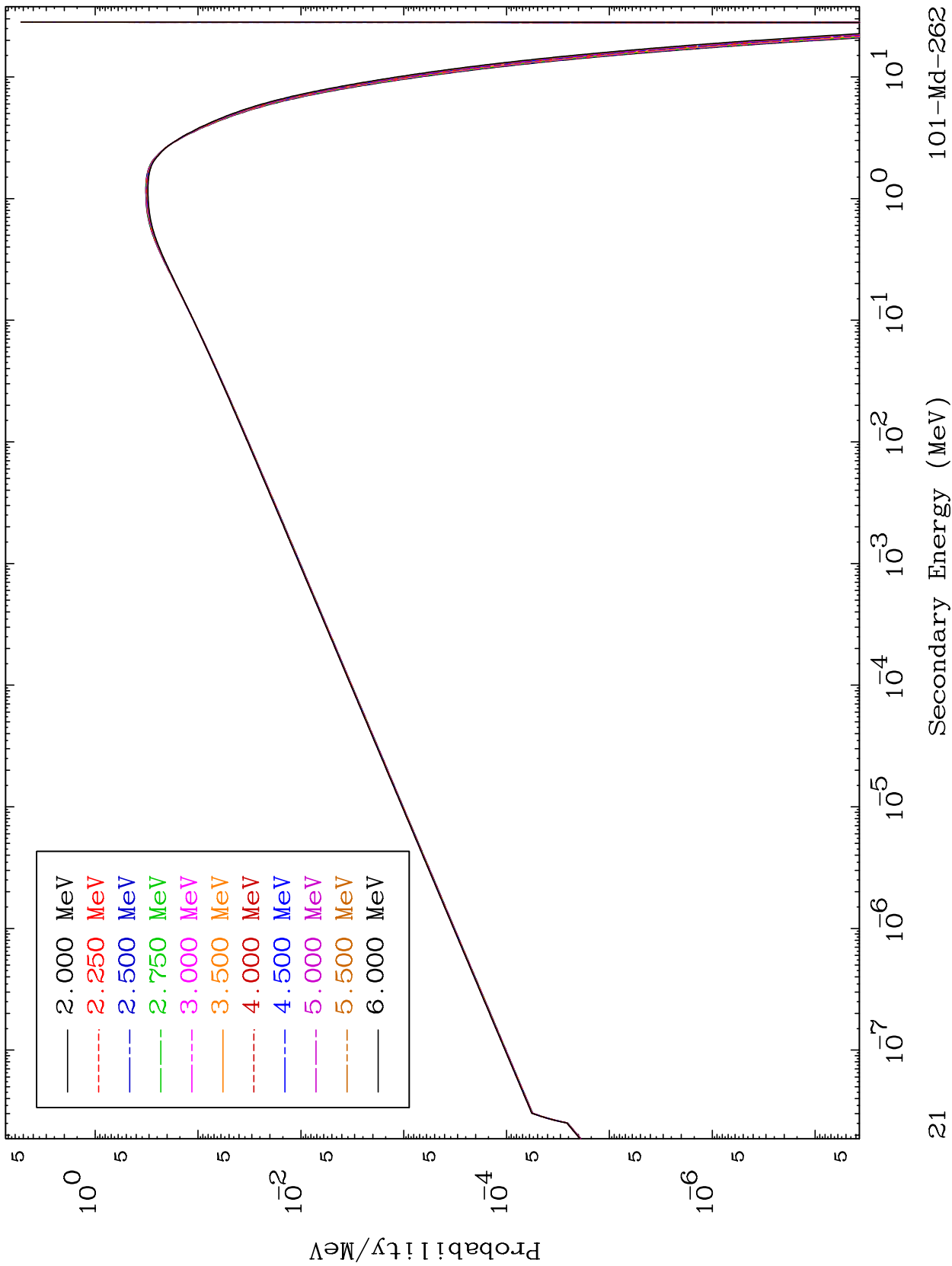


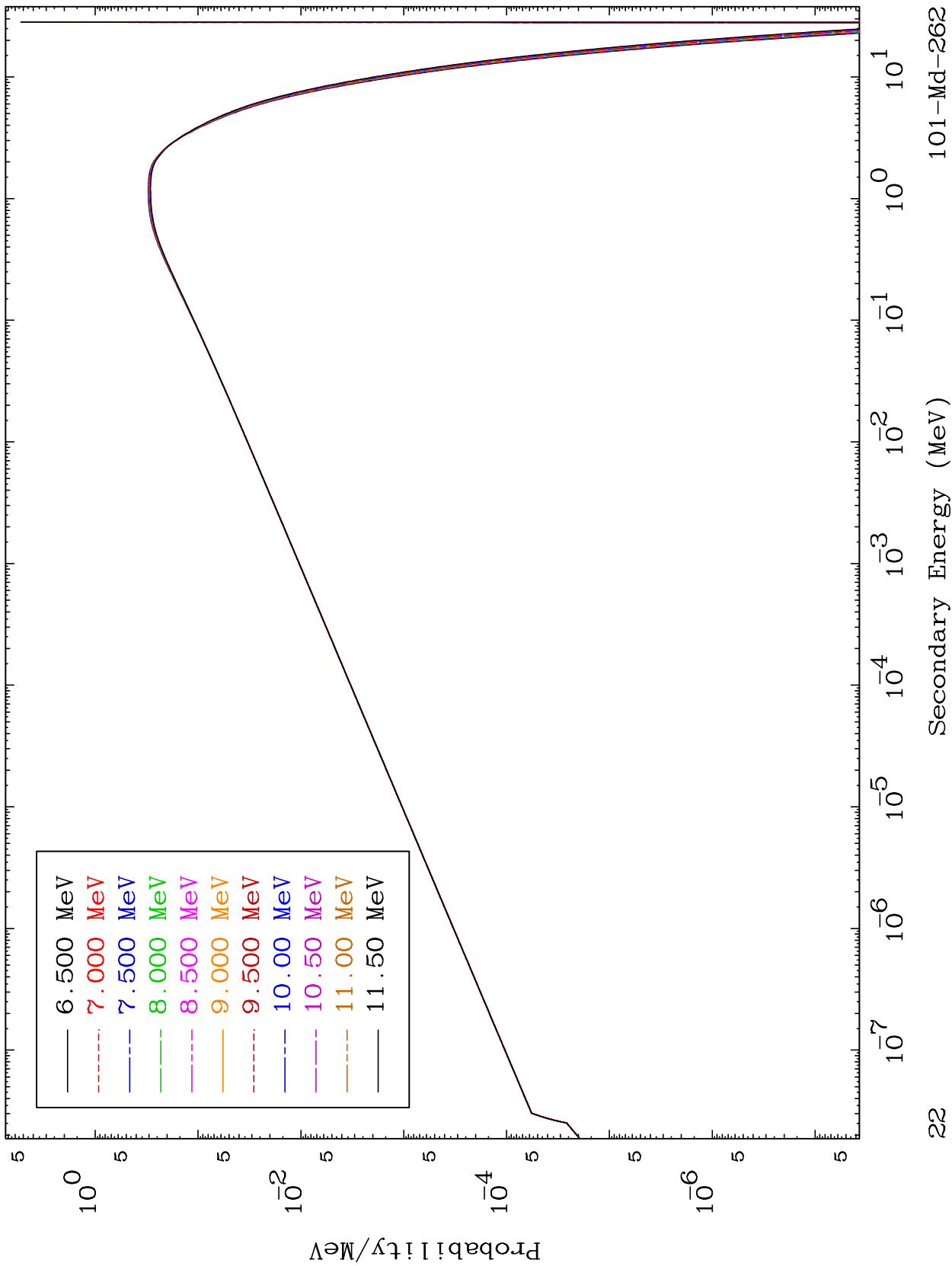
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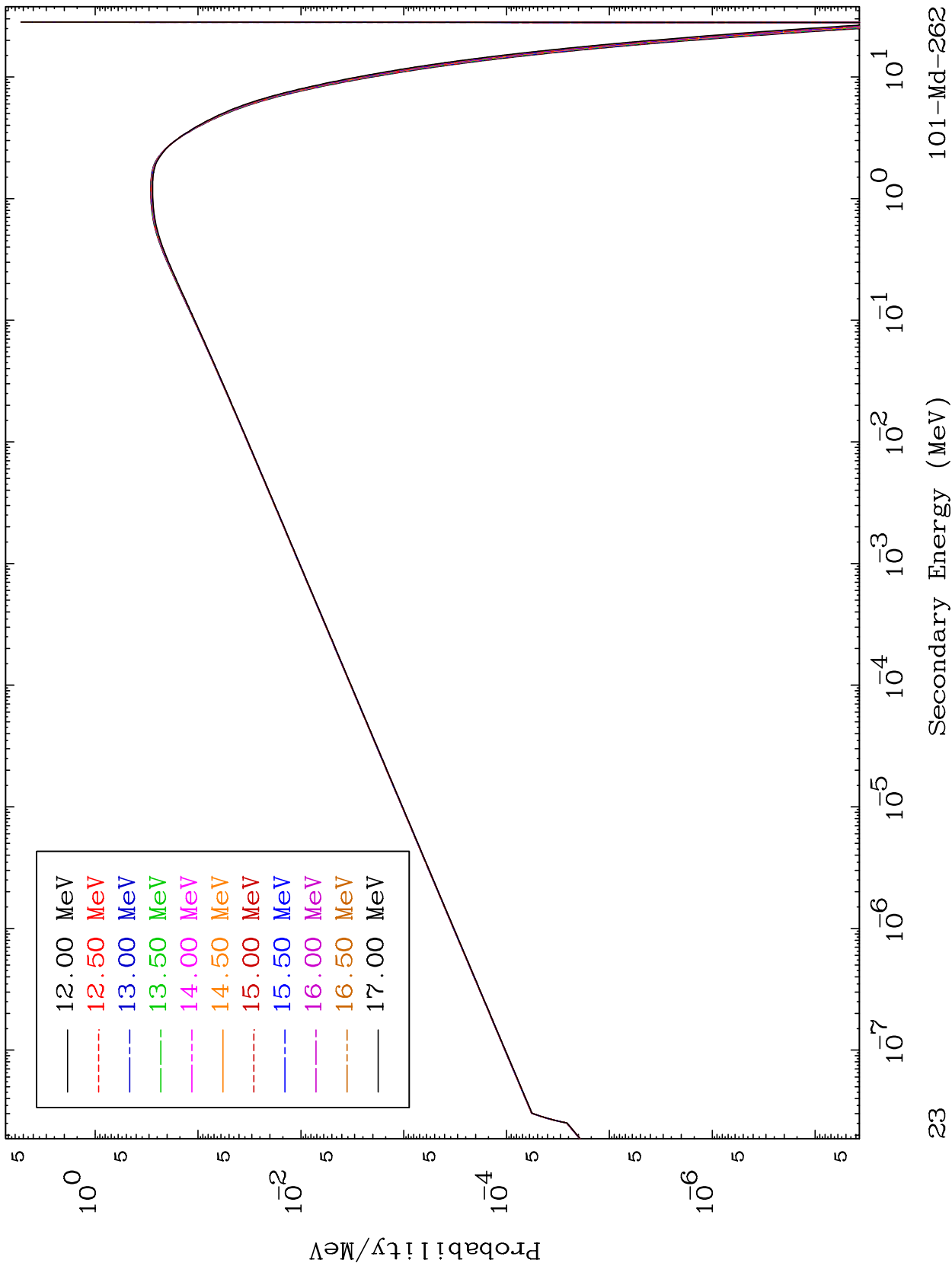
Incident Energy (MeV)

101-Md-262

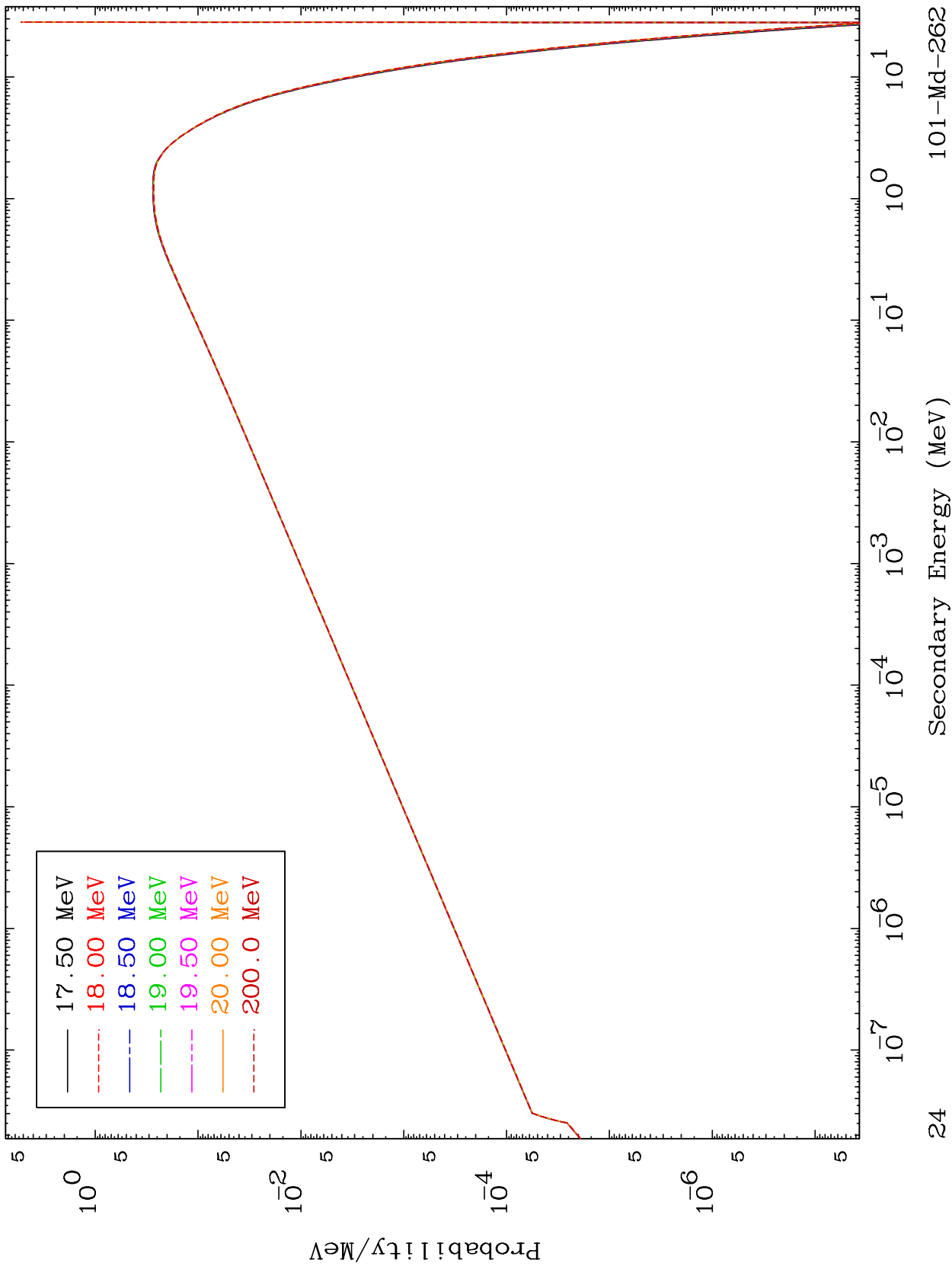








Fission
Energy Distribution

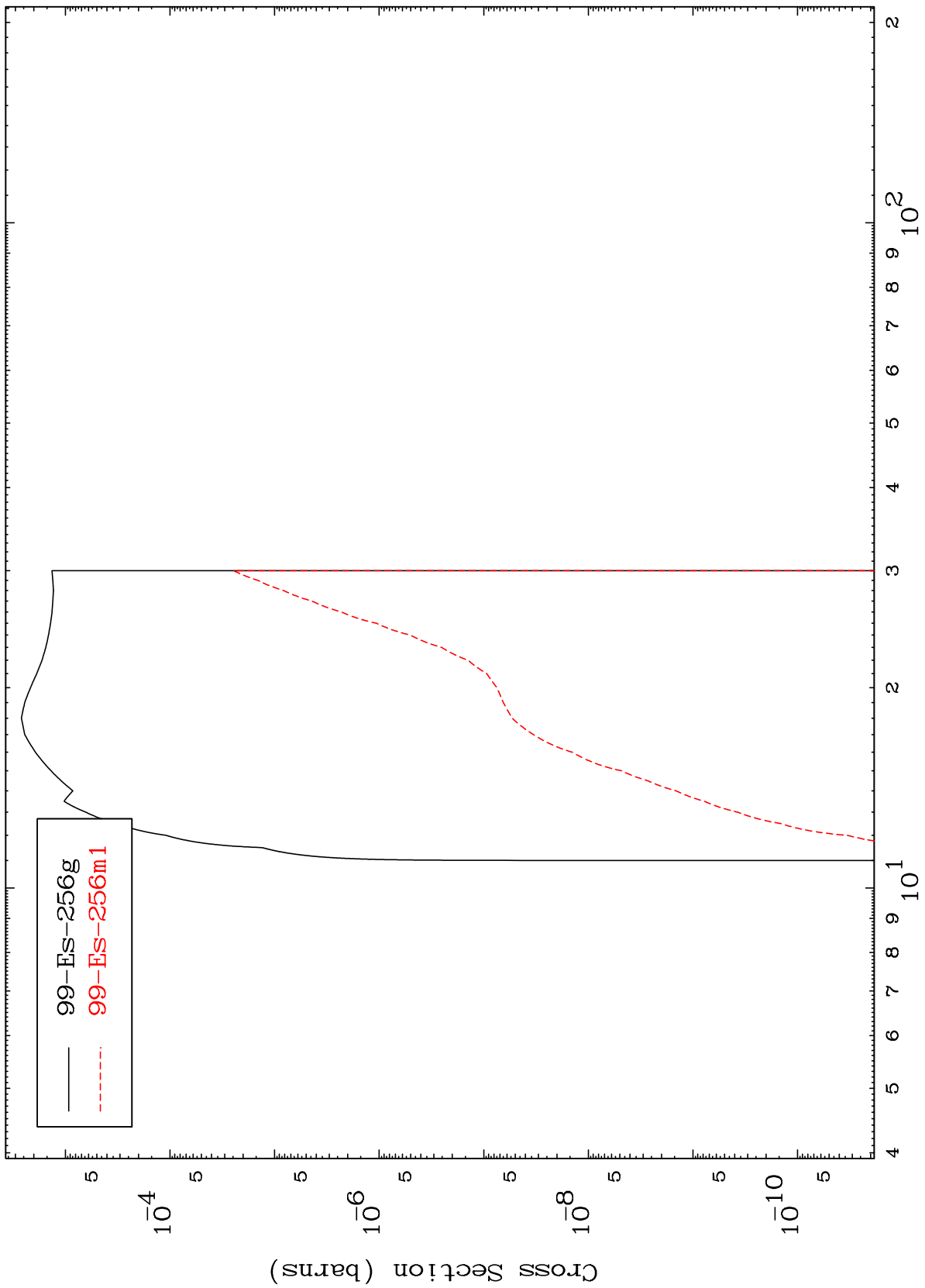


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(n,3n) α

101-Md-262

Radionuclide Production Cross Section



25

Incident Energy (MeV)

101-Md-262