

Program EVALPLOT
(Version 2015-2)

by

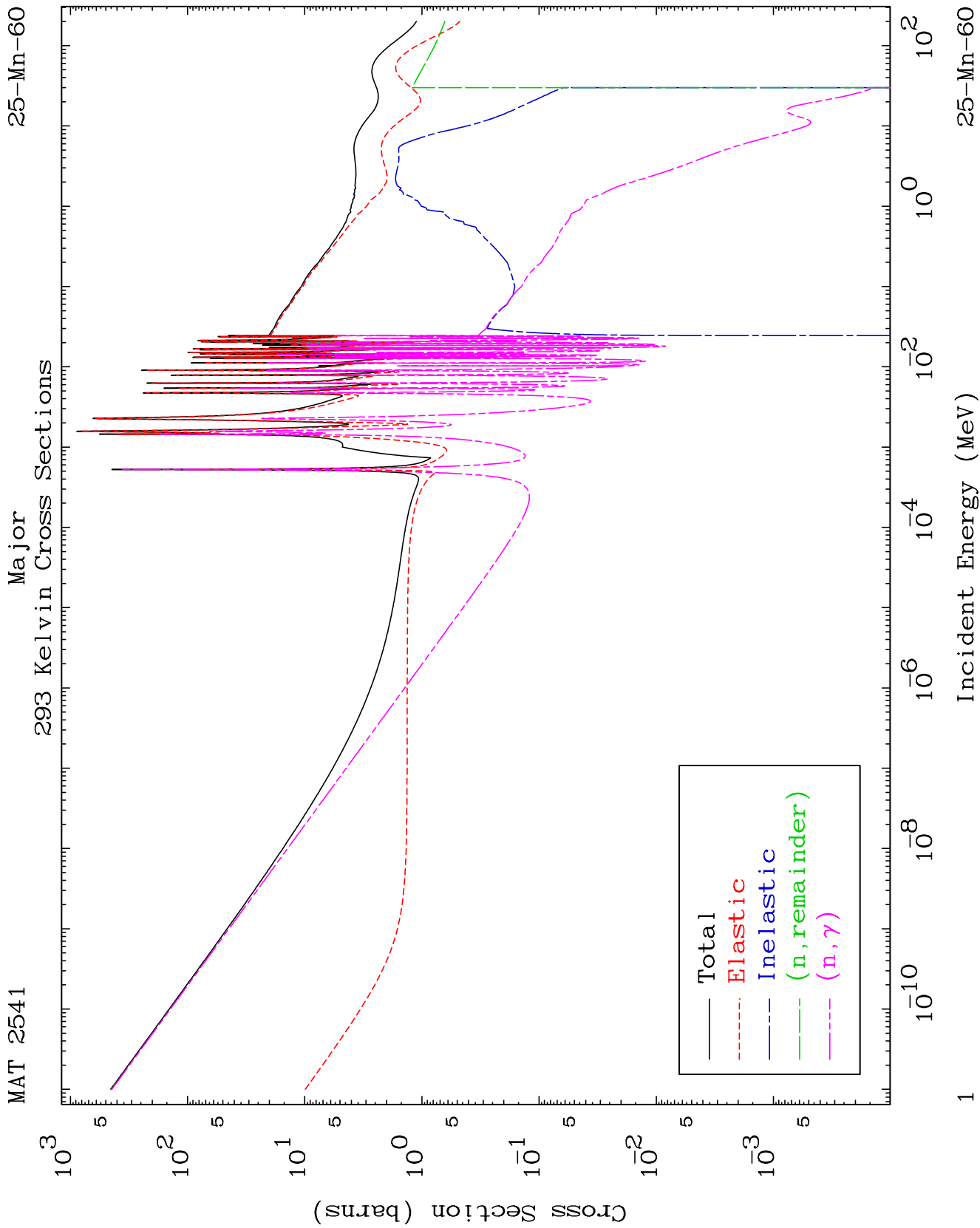
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(Present Contact Information)

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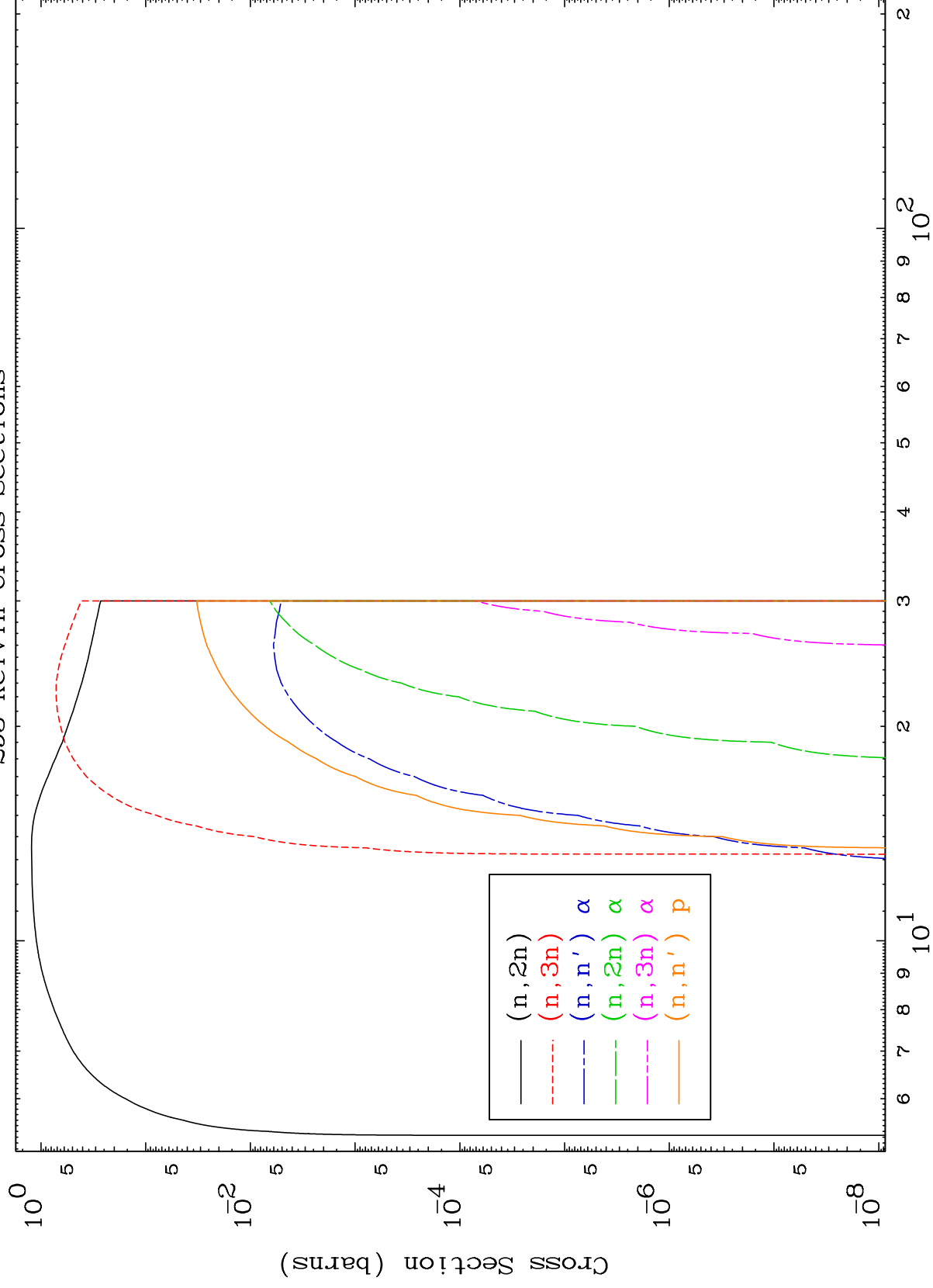
Press Mouse Button to Start

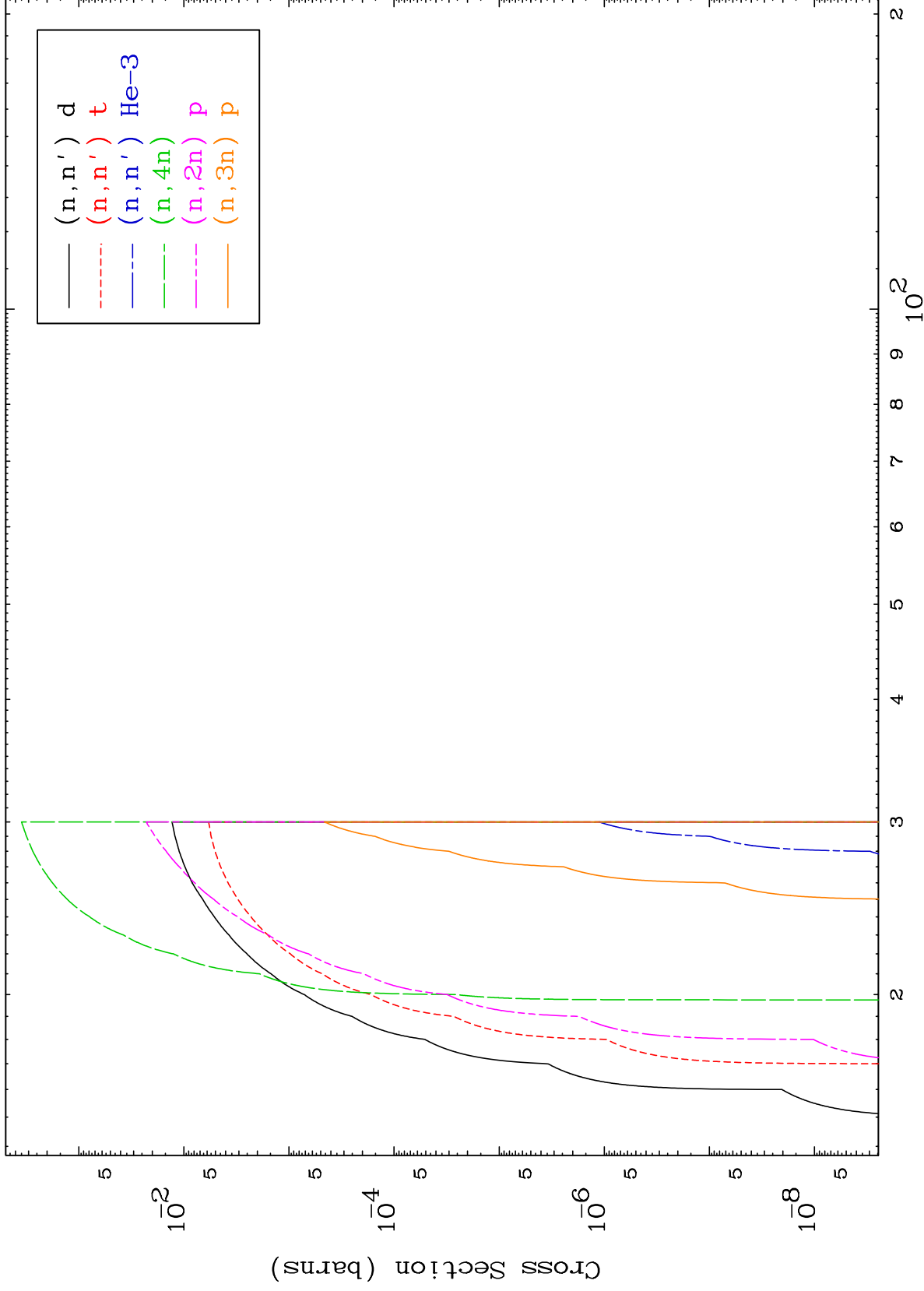


MAT 2541

Neutron Production
293 Kelvin Cross Sections

25-Mn-60

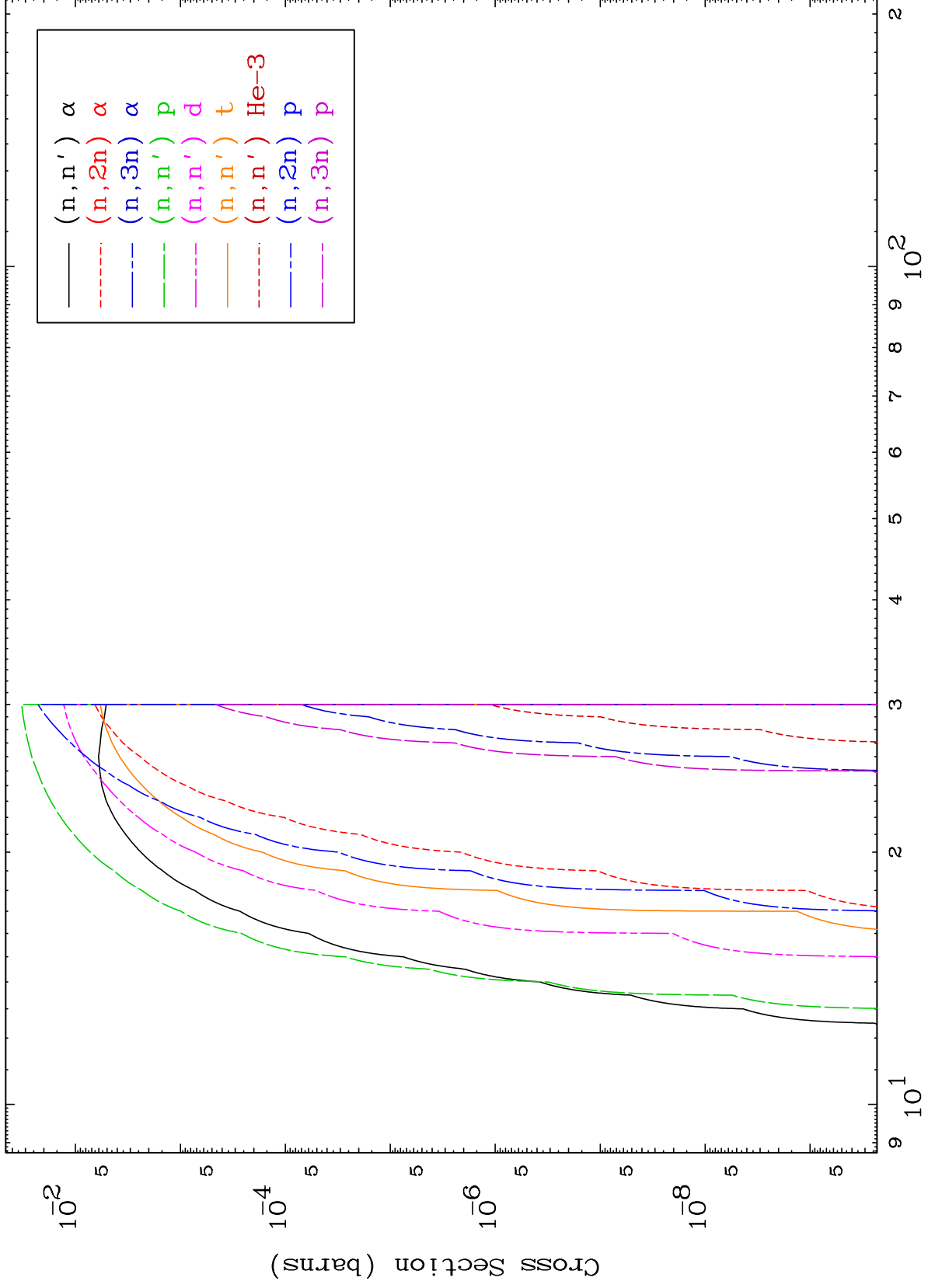




MAT 2541

Charged Particle
293 Kelvin Cross Sections

25-Mn-60



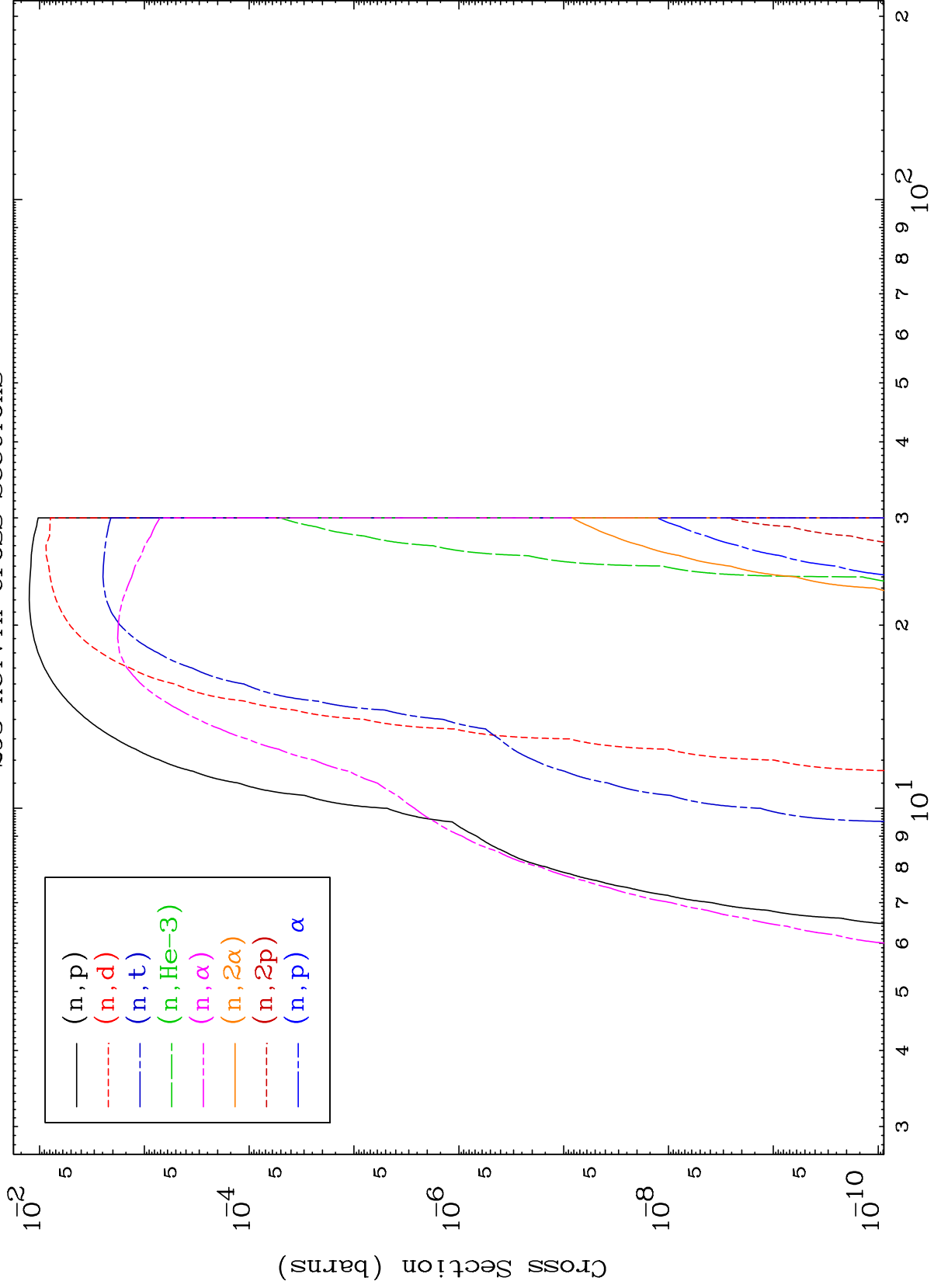
Incident Energy (MeV)

25-Mn-60

MAT 2541

Charged Particle
293 Kelvin Cross Sections

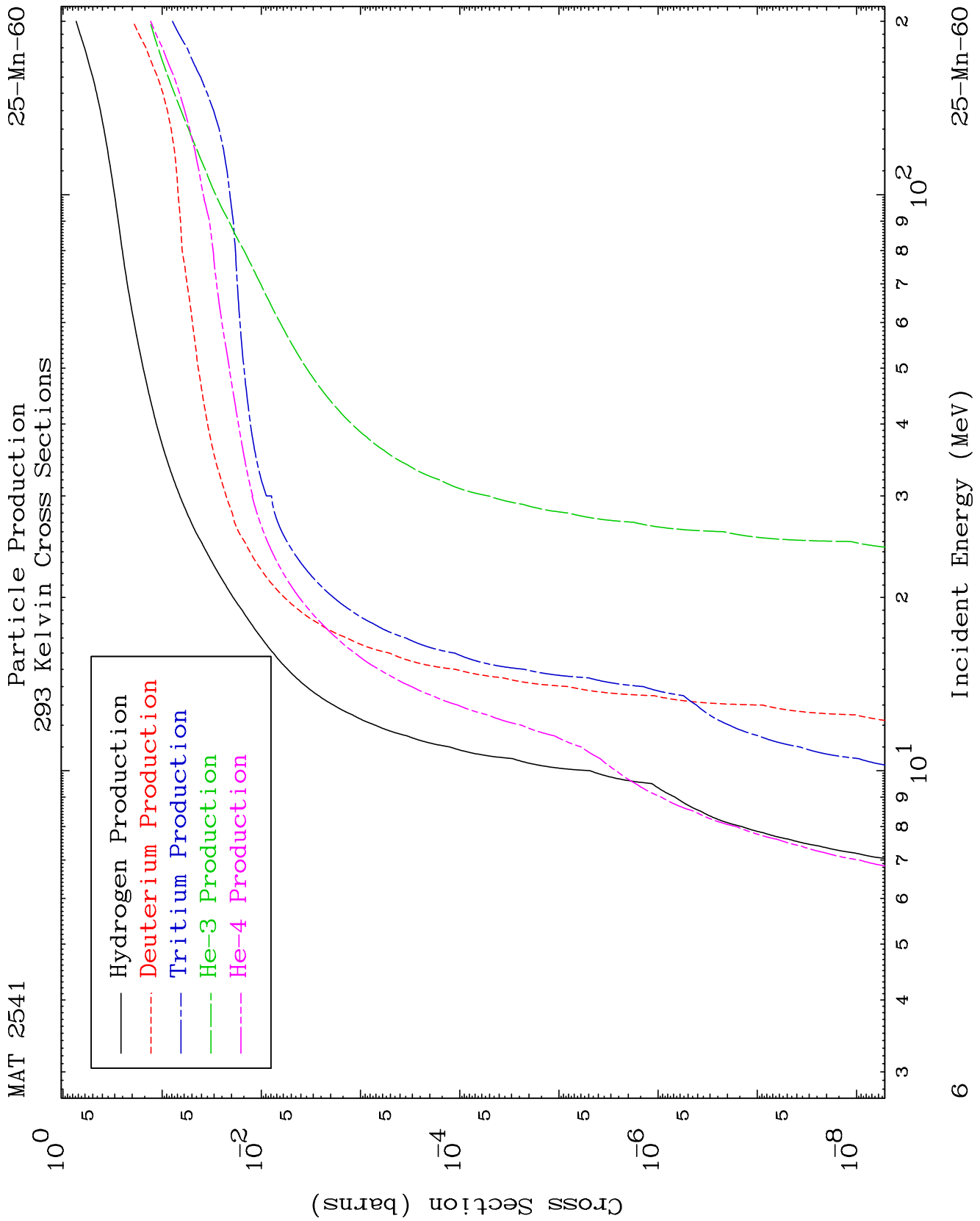
25-Mn-60



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Incident Energy (MeV)

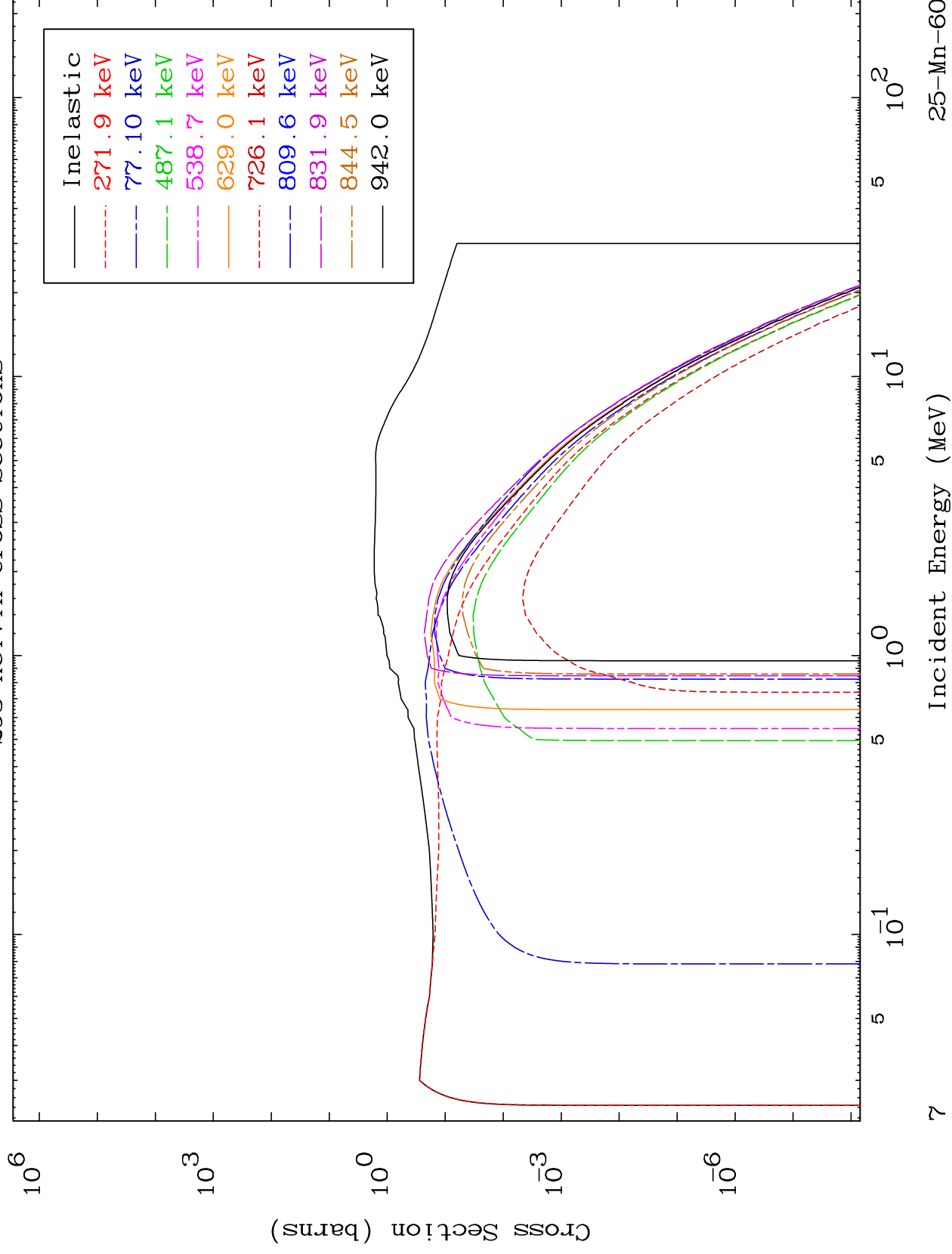
25-Mn-60



MAT 2541

(n,n') Level
293 Kelvin Cross Sections

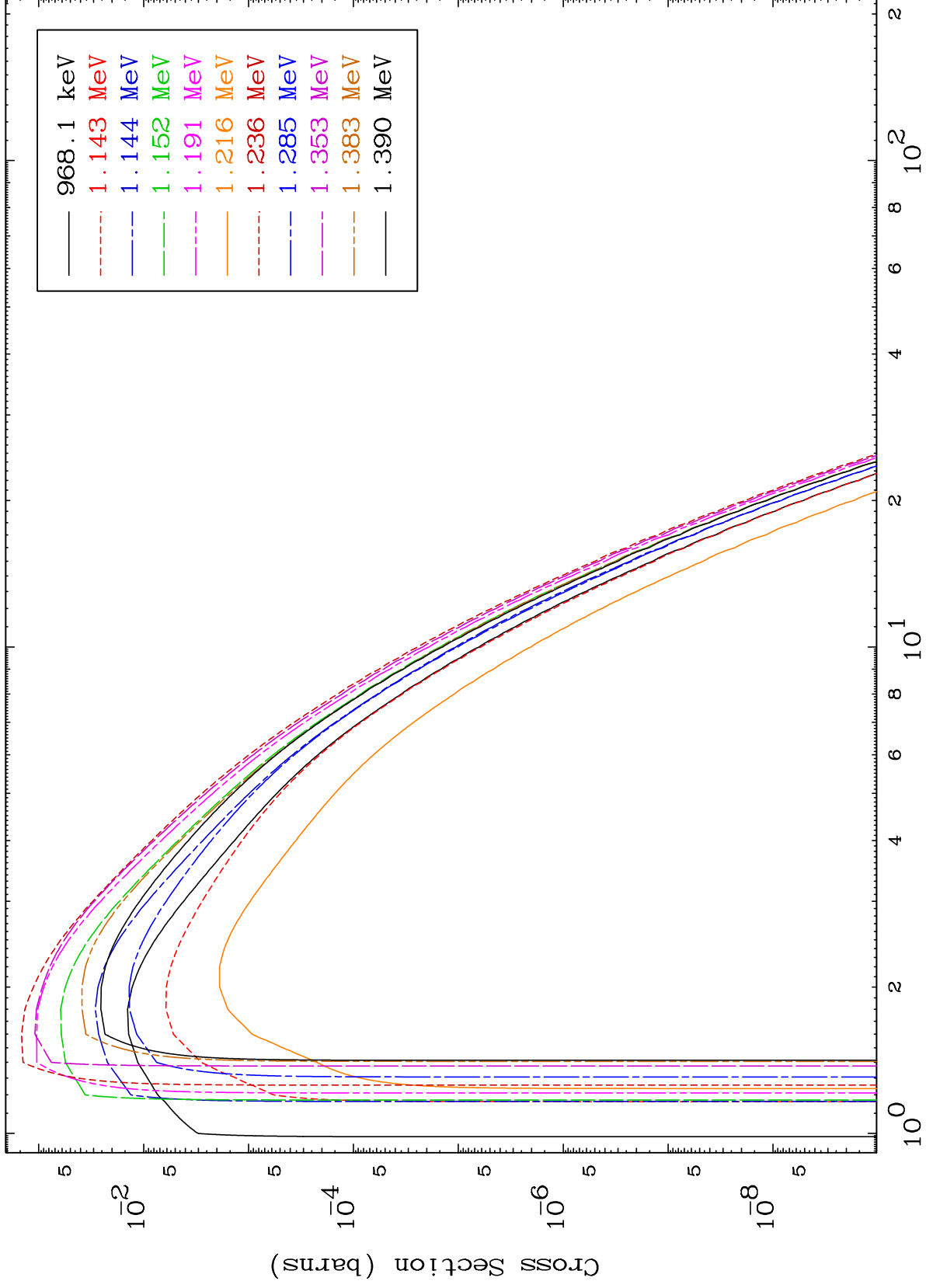
25-Mn-60



MAT 2541

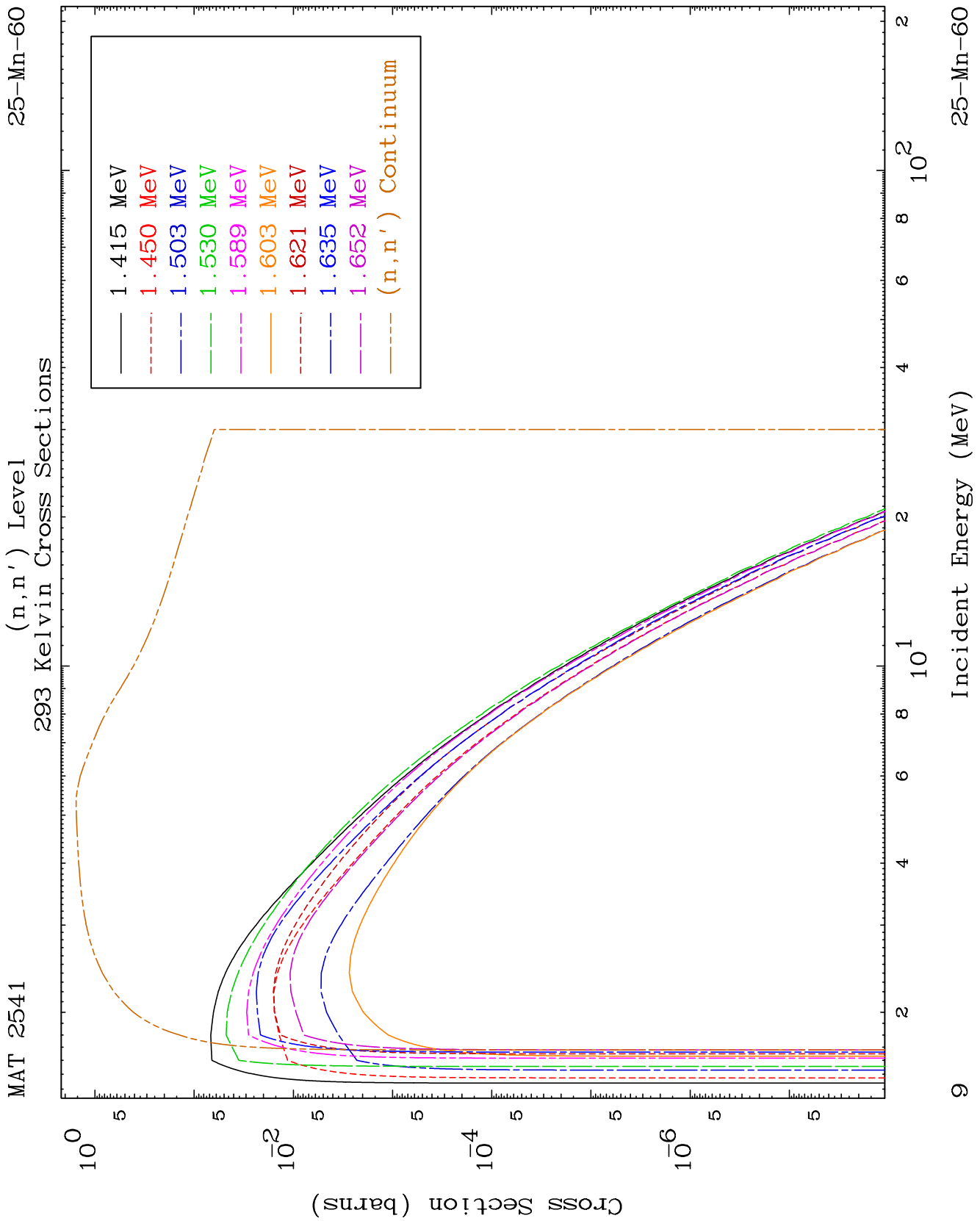
(n,n') Level
293 Kelvin Cross Sections

25-Mn-60



Incident Energy (MeV)

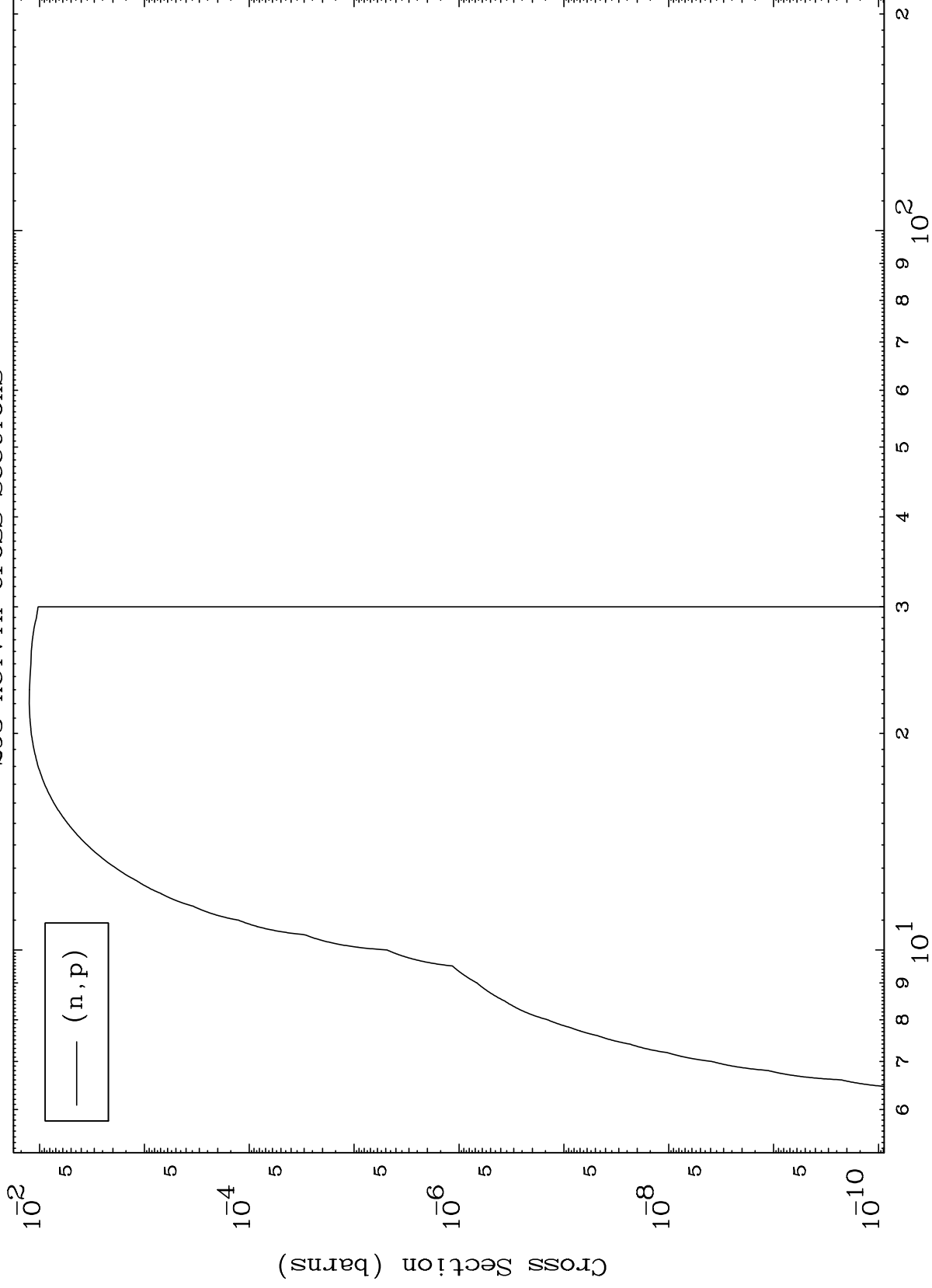
25-Mn-60



MAT 2541

(n,p) Levels
293 Kelvin Cross Sections

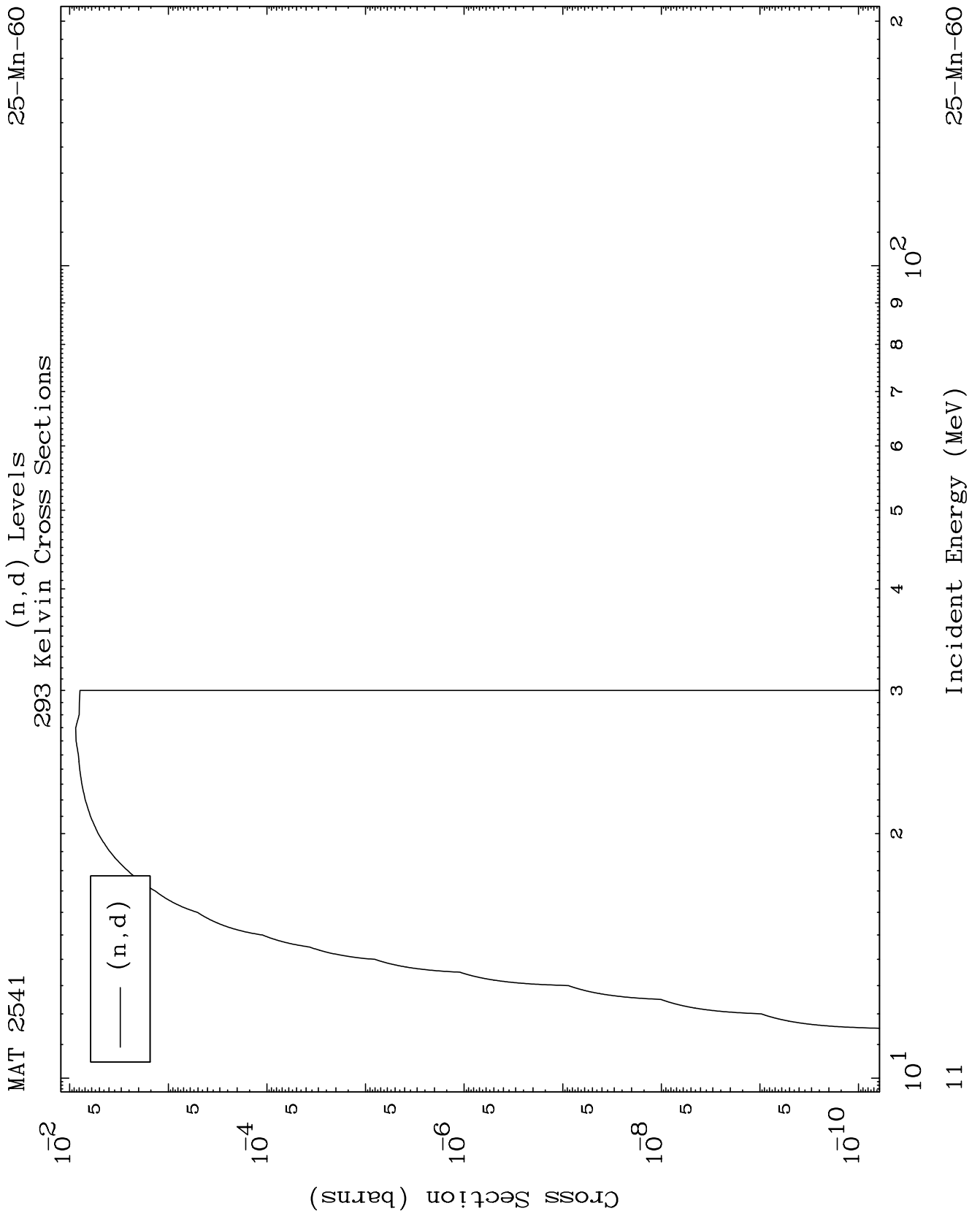
25-Mn-60



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Incident Energy (MeV)

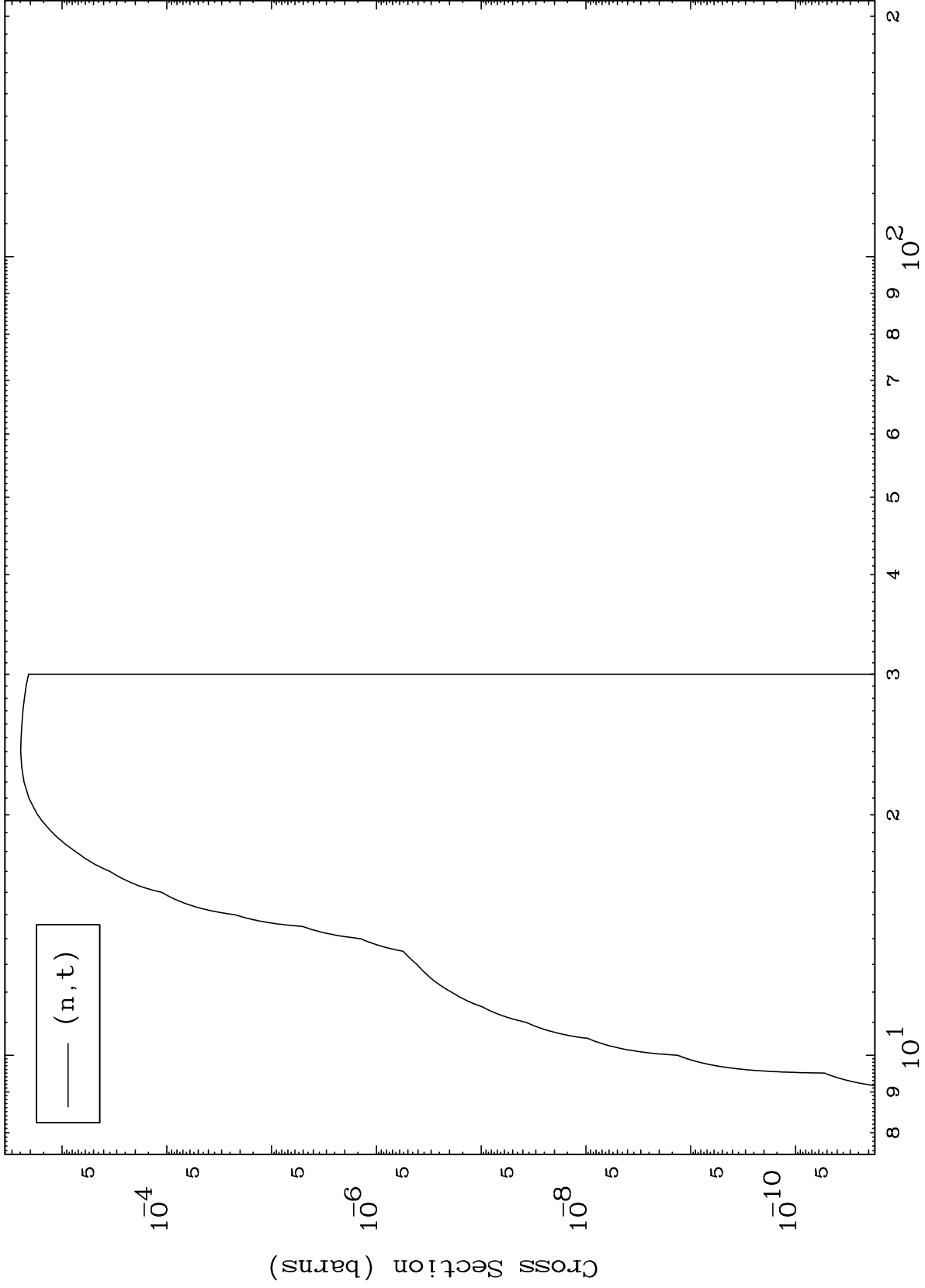
25-Mn-60



MAT 2541

(n,t) Levels
293 Kelvin Cross Sections

25-Mn-60



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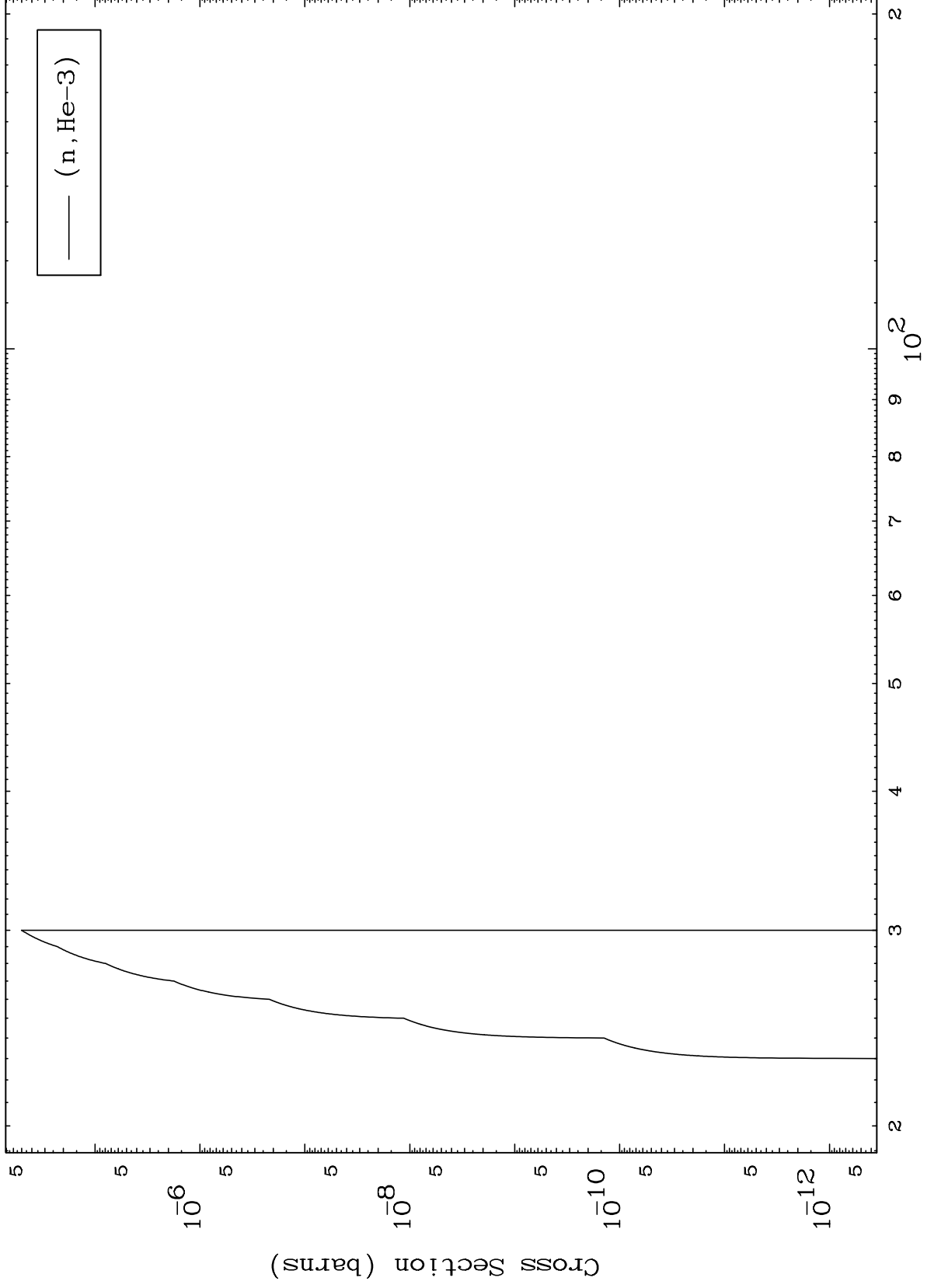
Incident Energy (MeV)

25-Mn-60

MAT 2541

(n,He3) Levels
293 Kelvin Cross Sections

25-Mn-60



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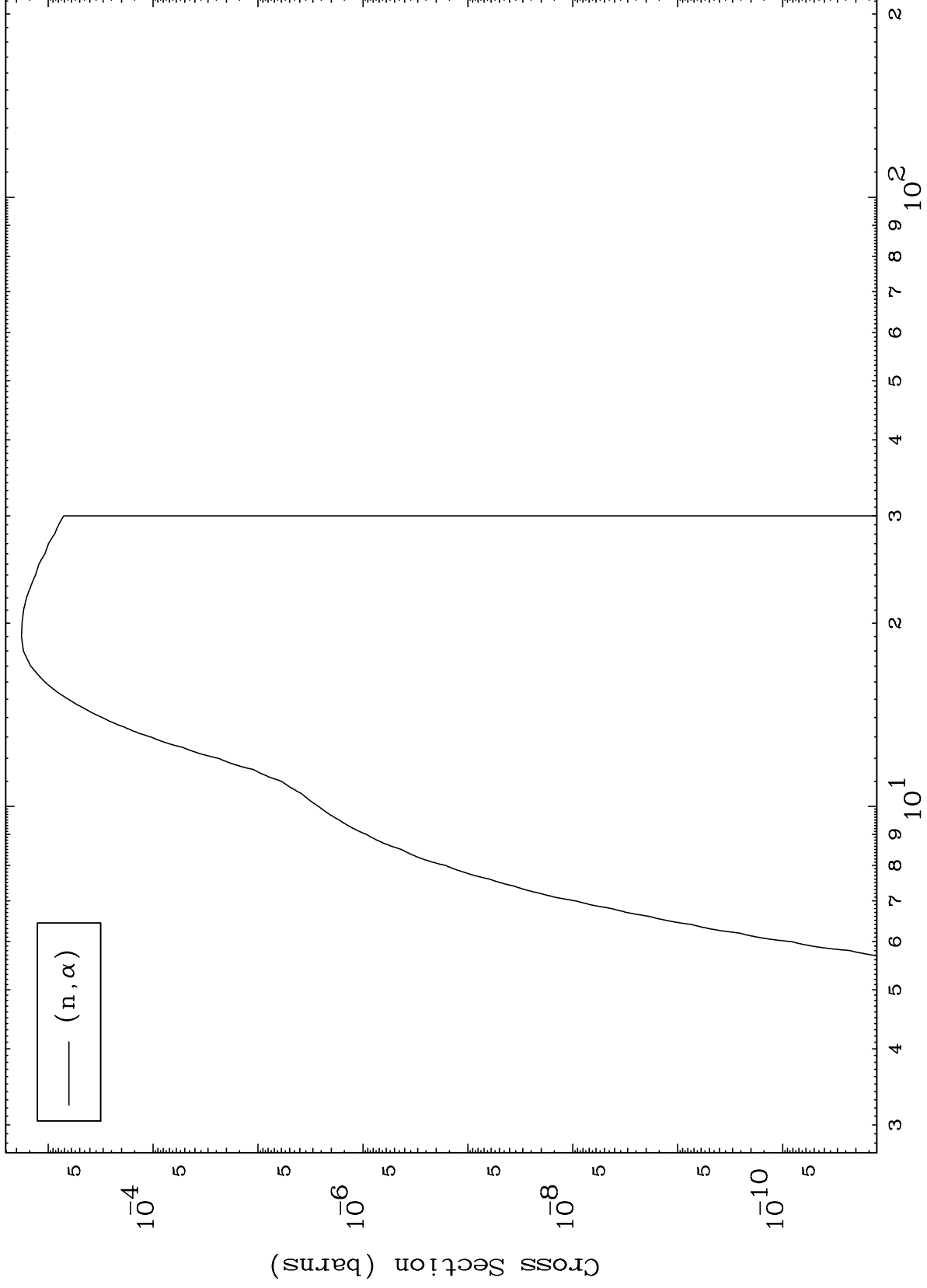
Incident Energy (MeV)

25-Mn-60

MAT 2541

(n, α) Levels
293 Kelvin Cross Sections

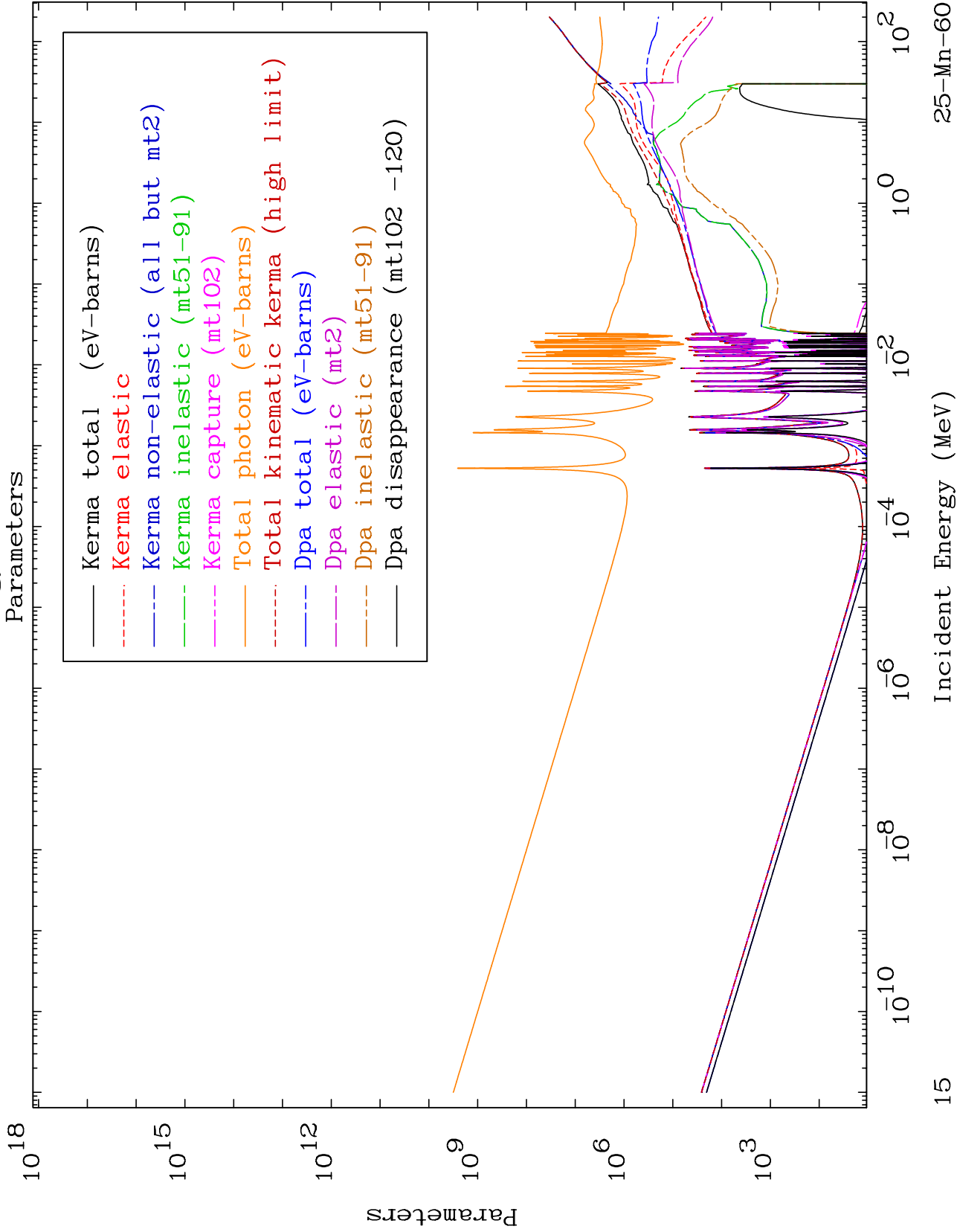
25-Mn-60

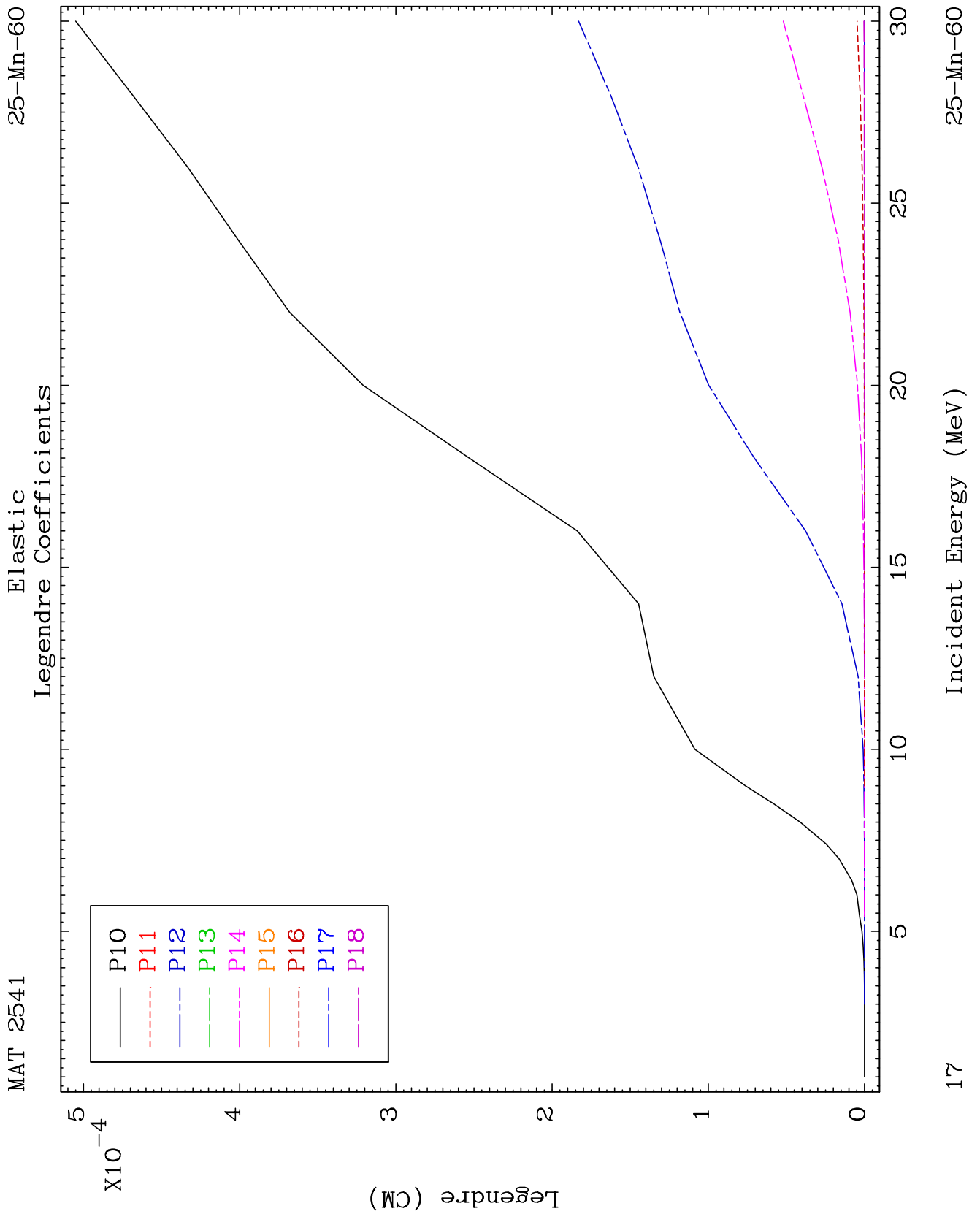


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Incident Energy (MeV)

25-Mn-60

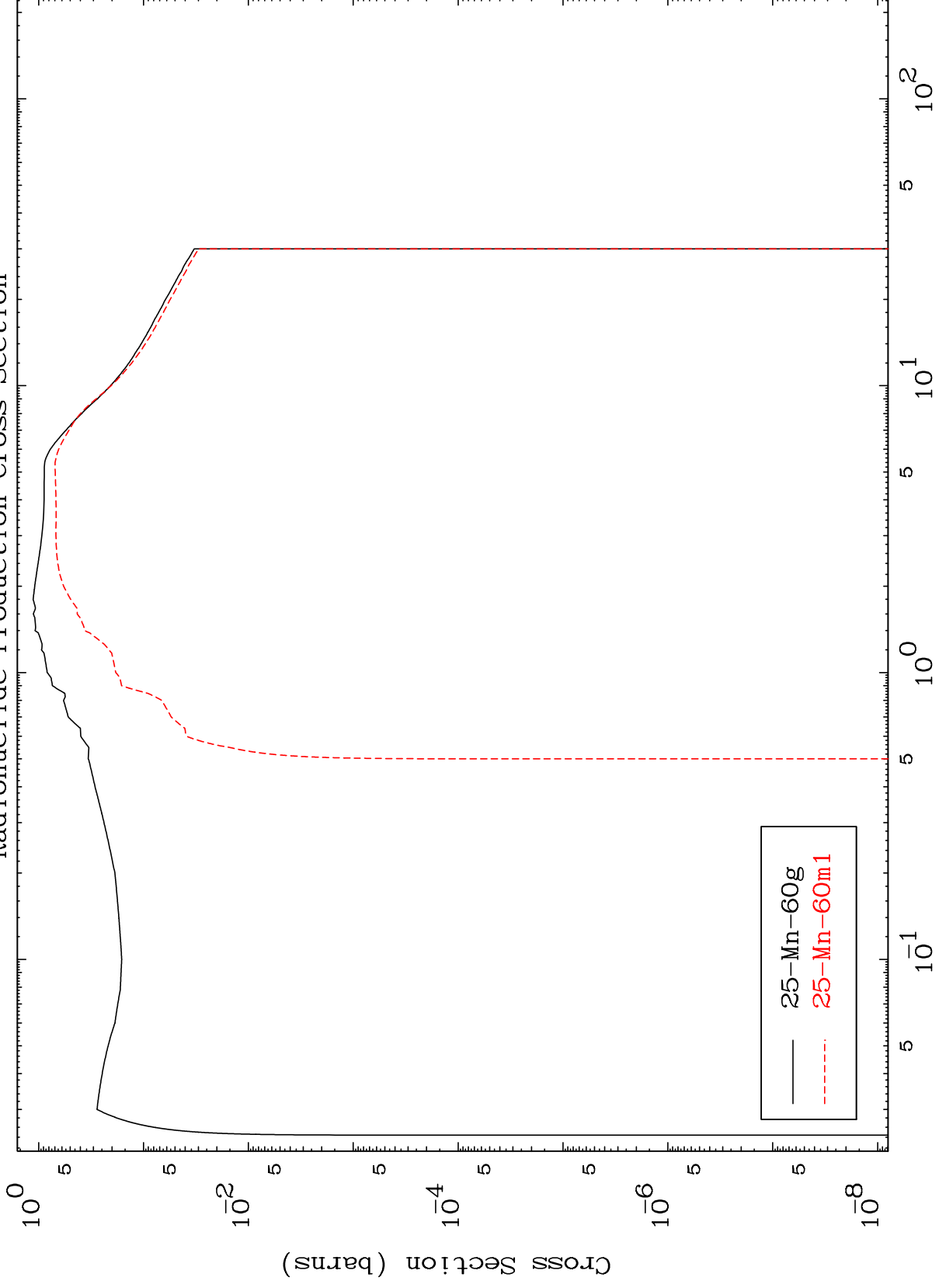




MAT 2541

25-Mn-60

Inelastic
Radionuclide Production Cross Section



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25-Mn-60

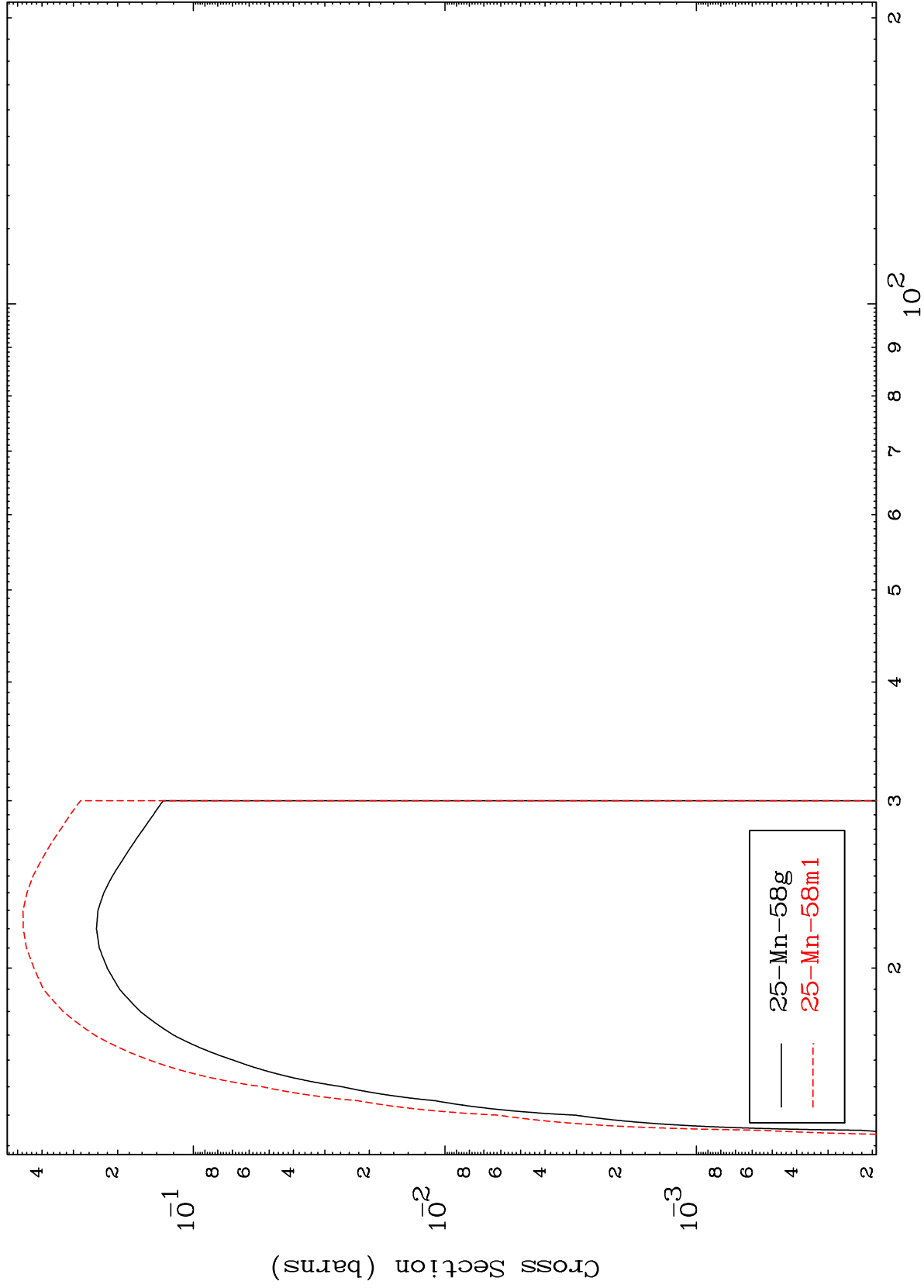
Incident Energy (MeV)

MAT 2541

(n,3n)

25-Mn-60

Radionuclide Production Cross Section



20

Incident Energy (MeV)

25-Mn-60