

Program EVALPLOT  
(Version 2015-2)

by

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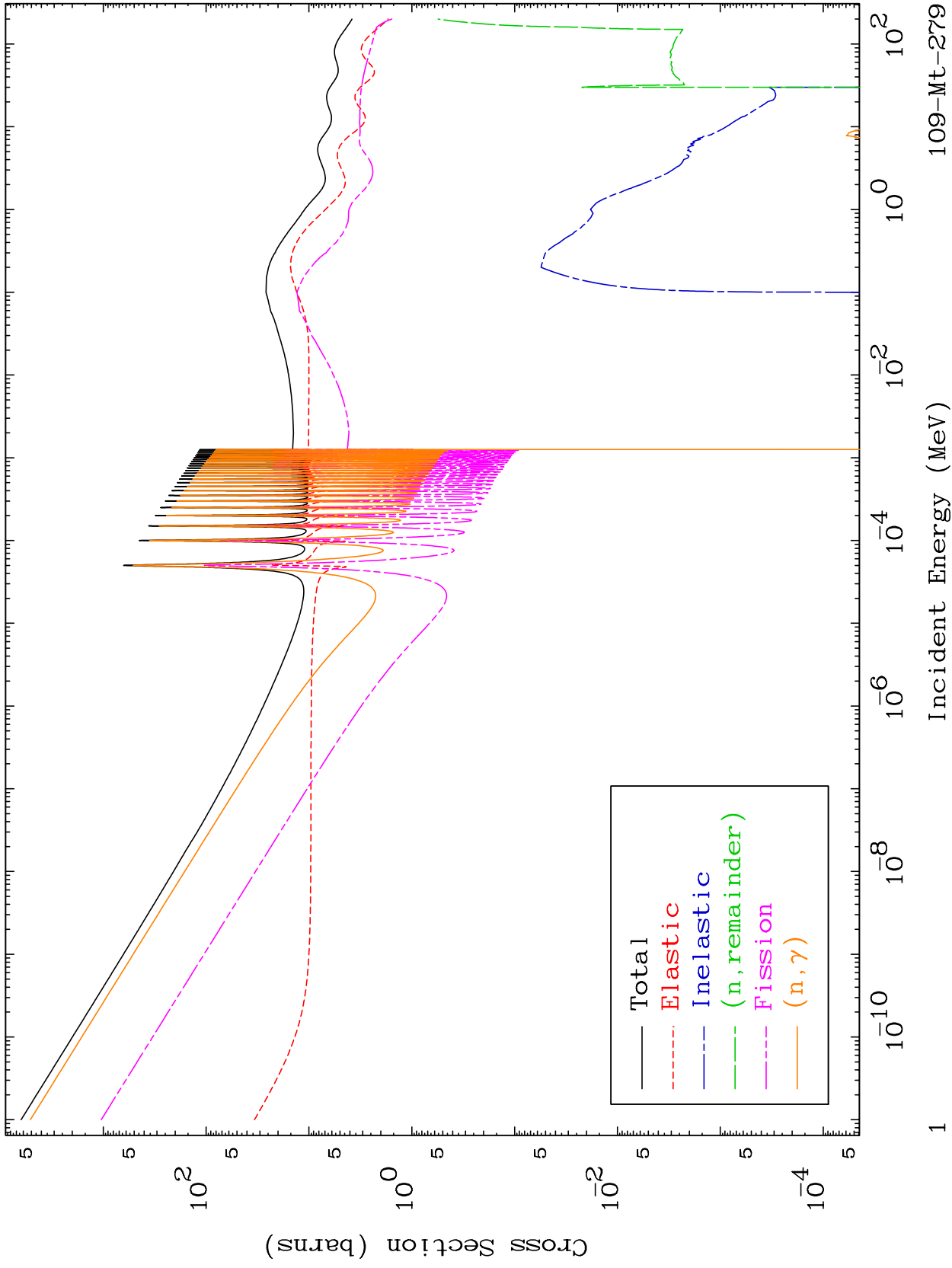
E.Mail:redcullen1@comcast.net  
Web:home.comcast.net/~redcullen1

Press Mouse Button to Start

MAT 979

Major  
293 Kelvin Cross Sections

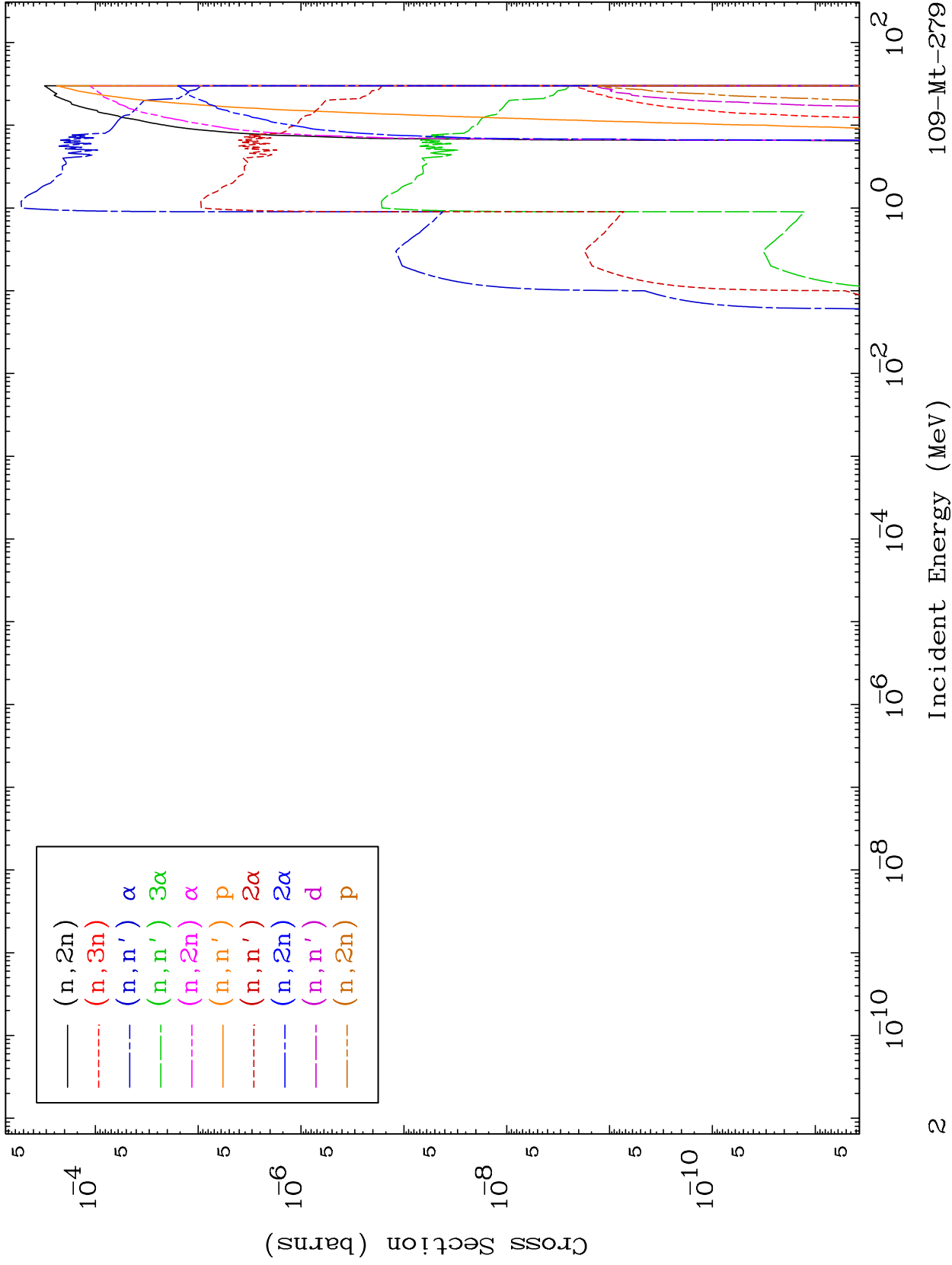
109-Mt-279



MAT 979

Neutron Production  
293 Kelvin Cross Sections

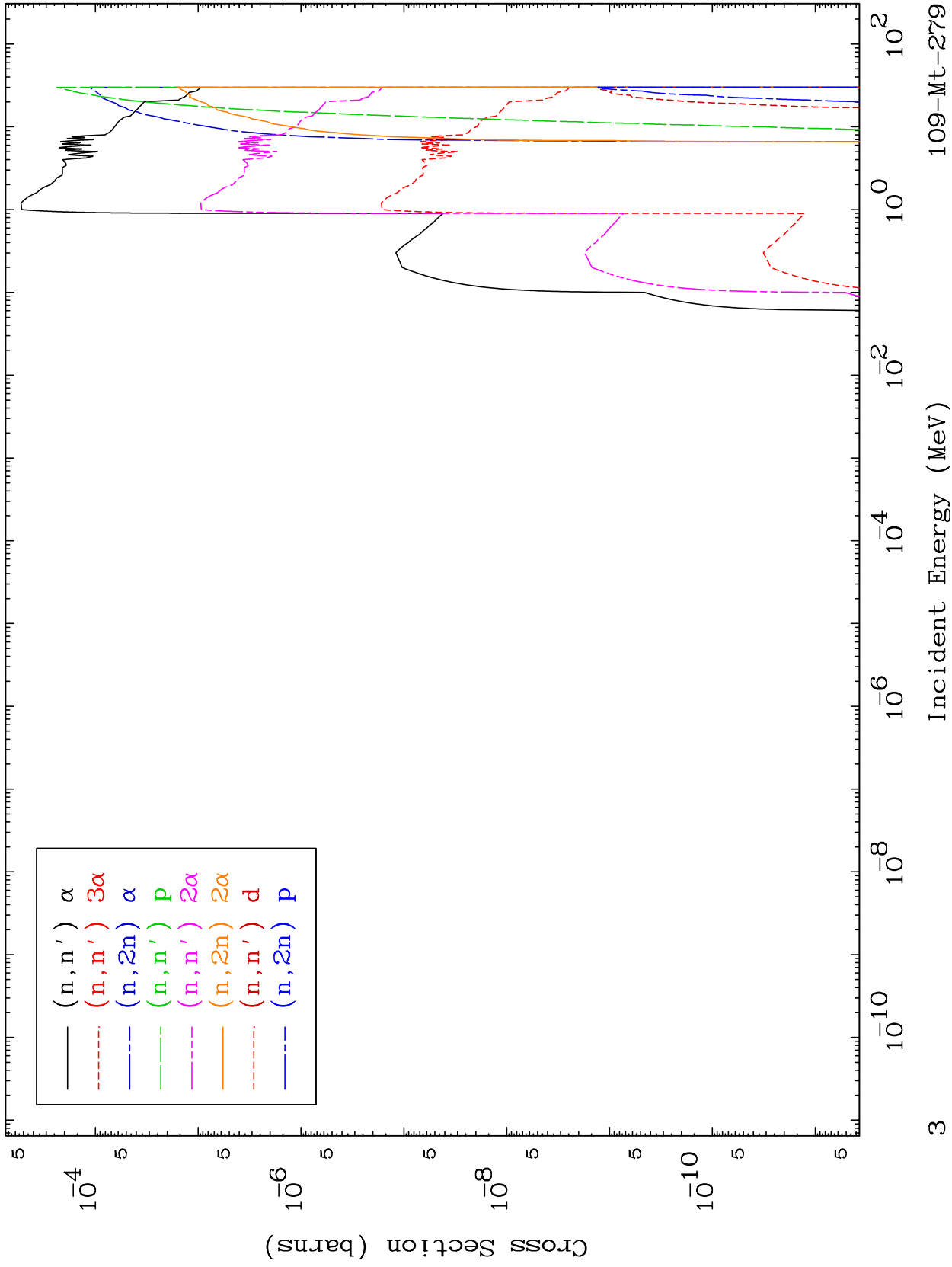
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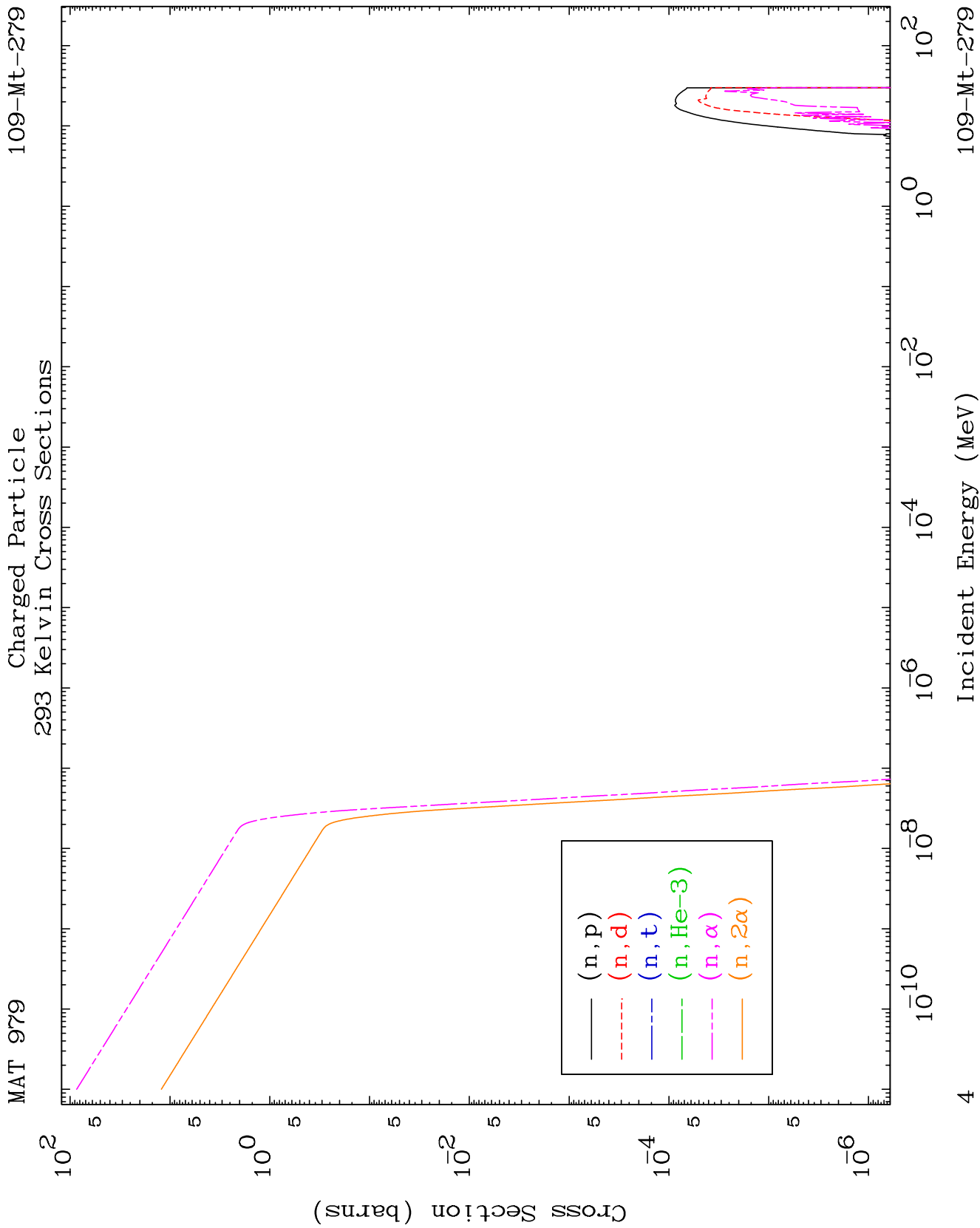


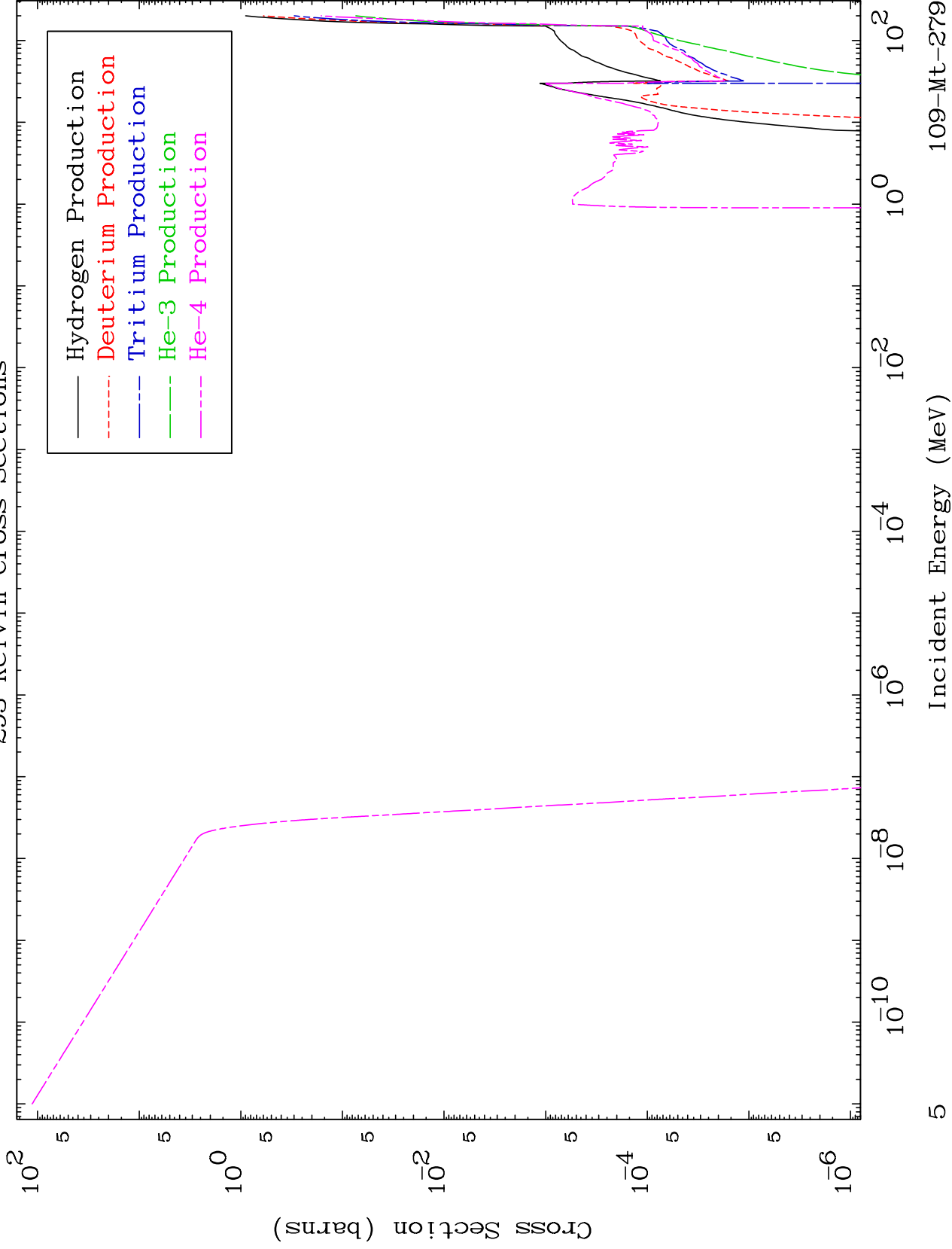
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Charged Particle  
293 Kelvin Cross Sections

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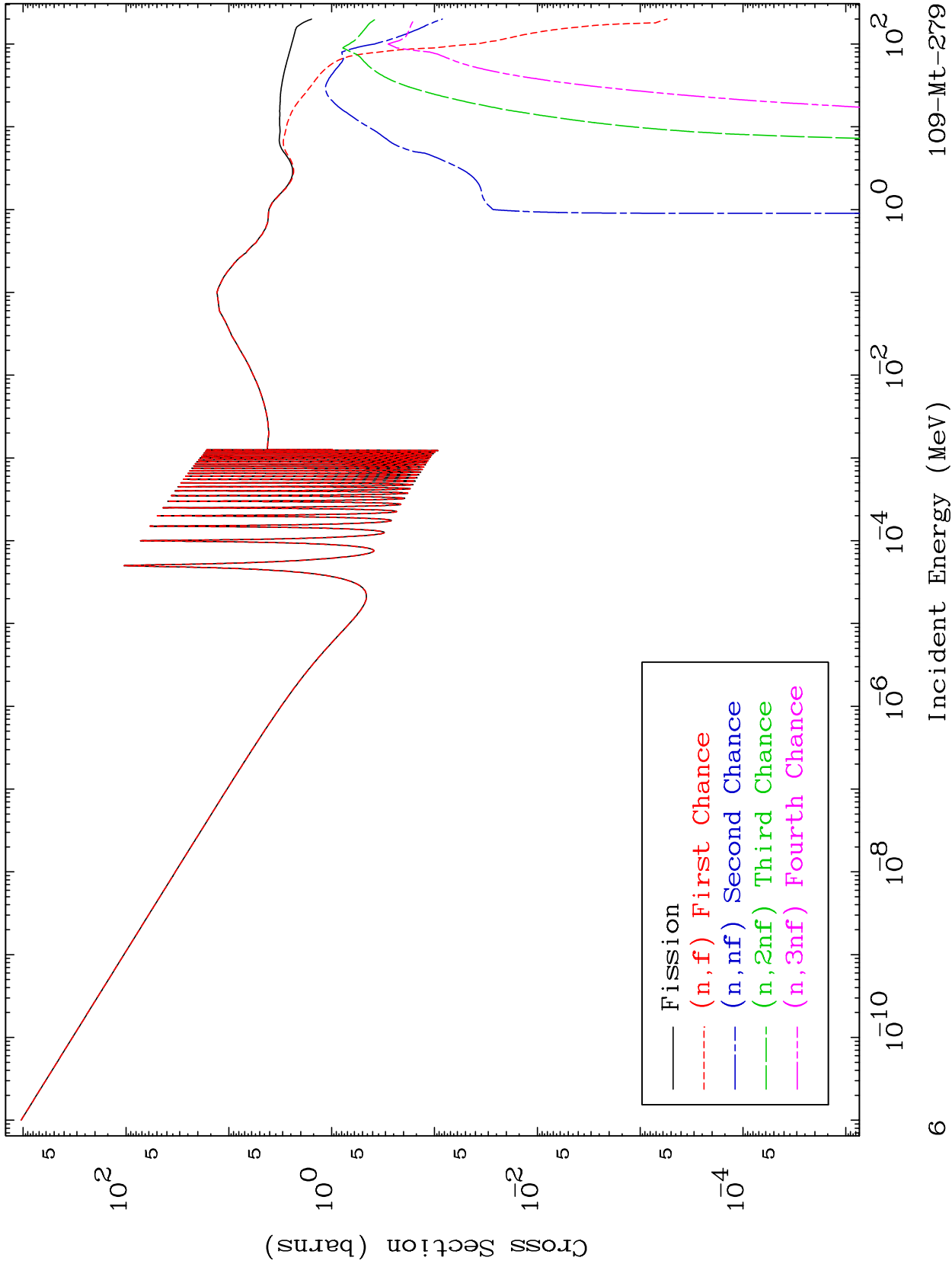




MAT 979

Fission  
293 Kelvin Cross Sections

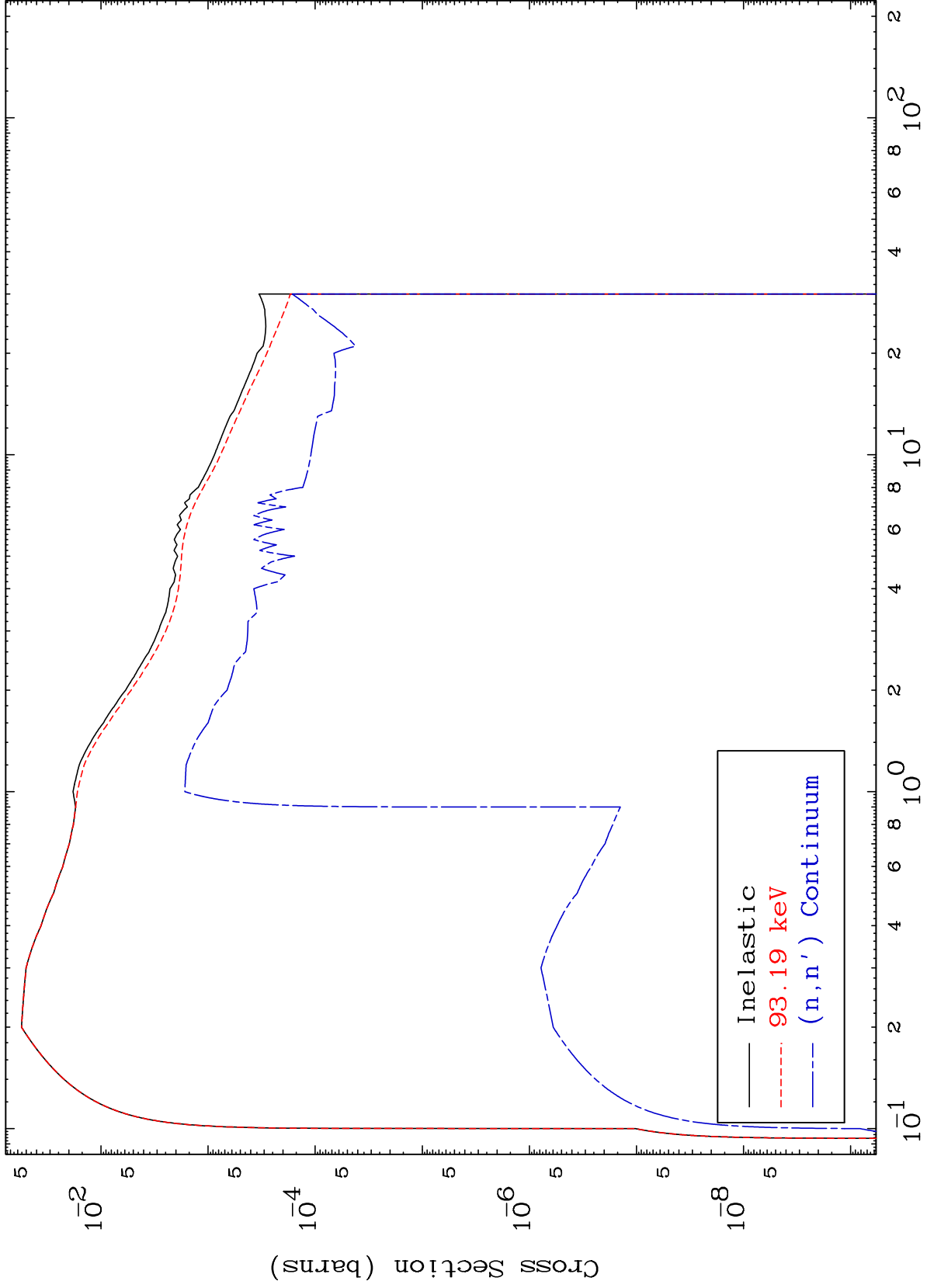
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(n,n') Level  
293 Kelvin Cross Sections

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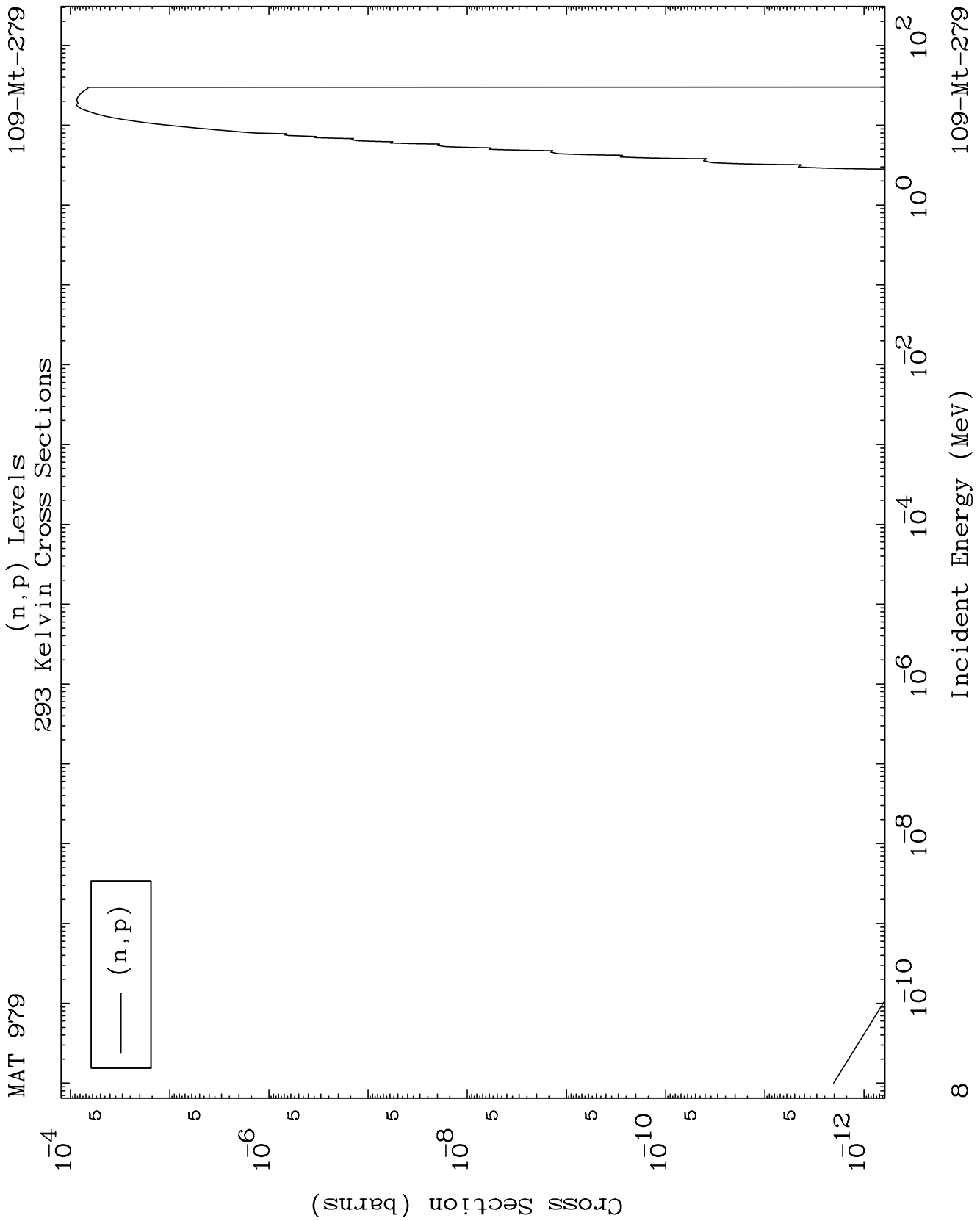


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109-Mt-279

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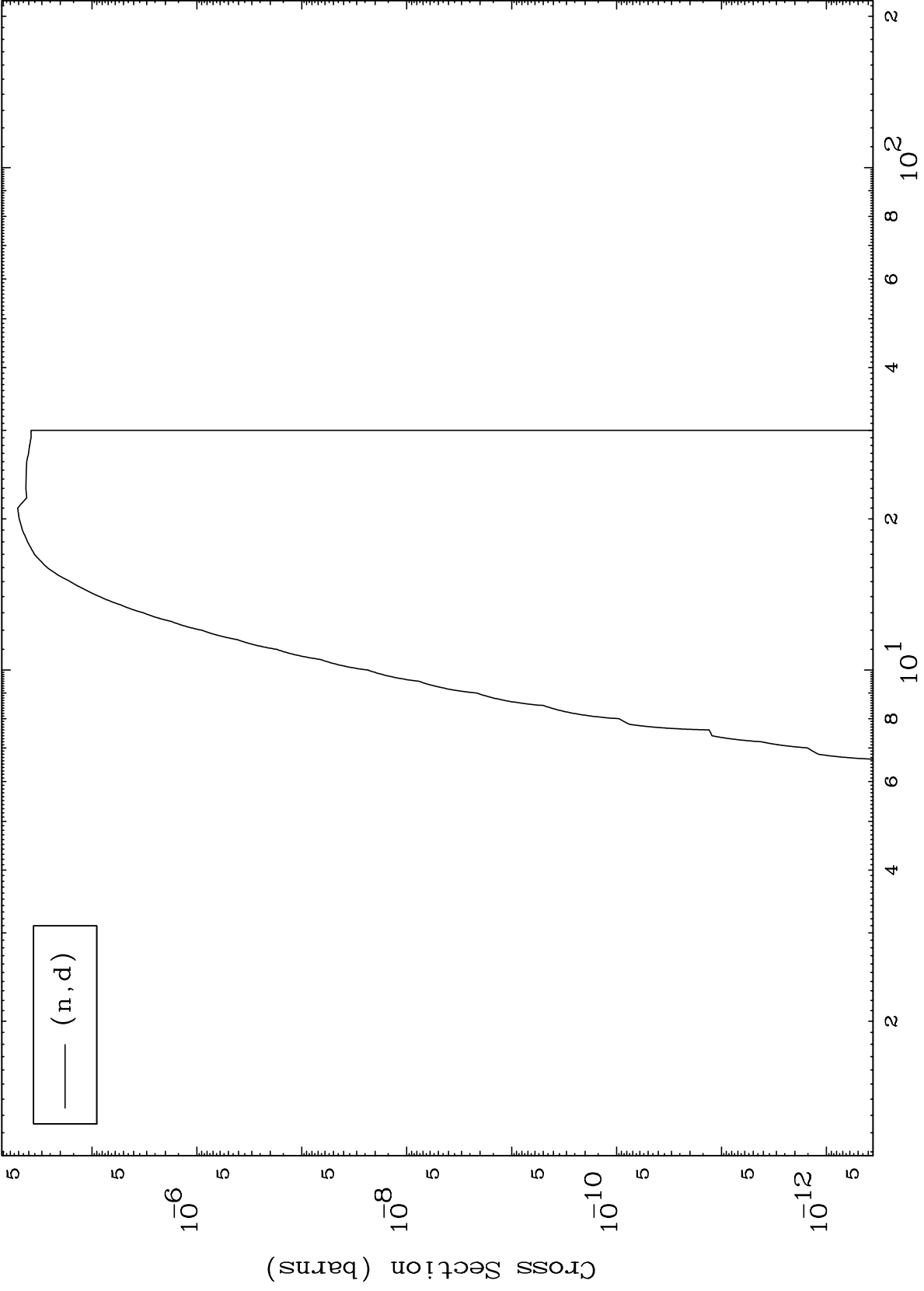




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(n,d) Levels  
293 Kelvin Cross Sections

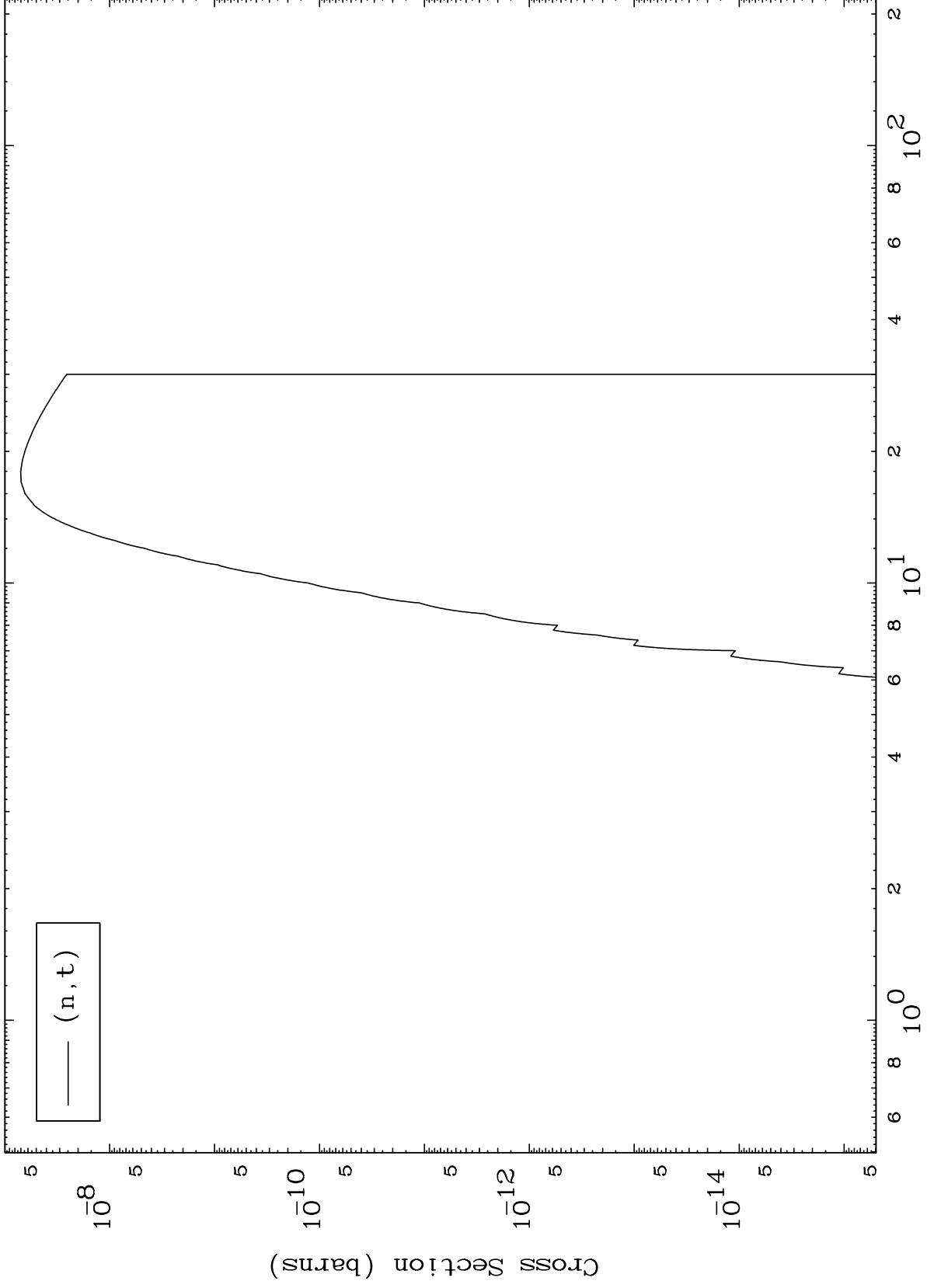
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(n,t) Levels  
293 Kelvin Cross Sections

109-Mt-279



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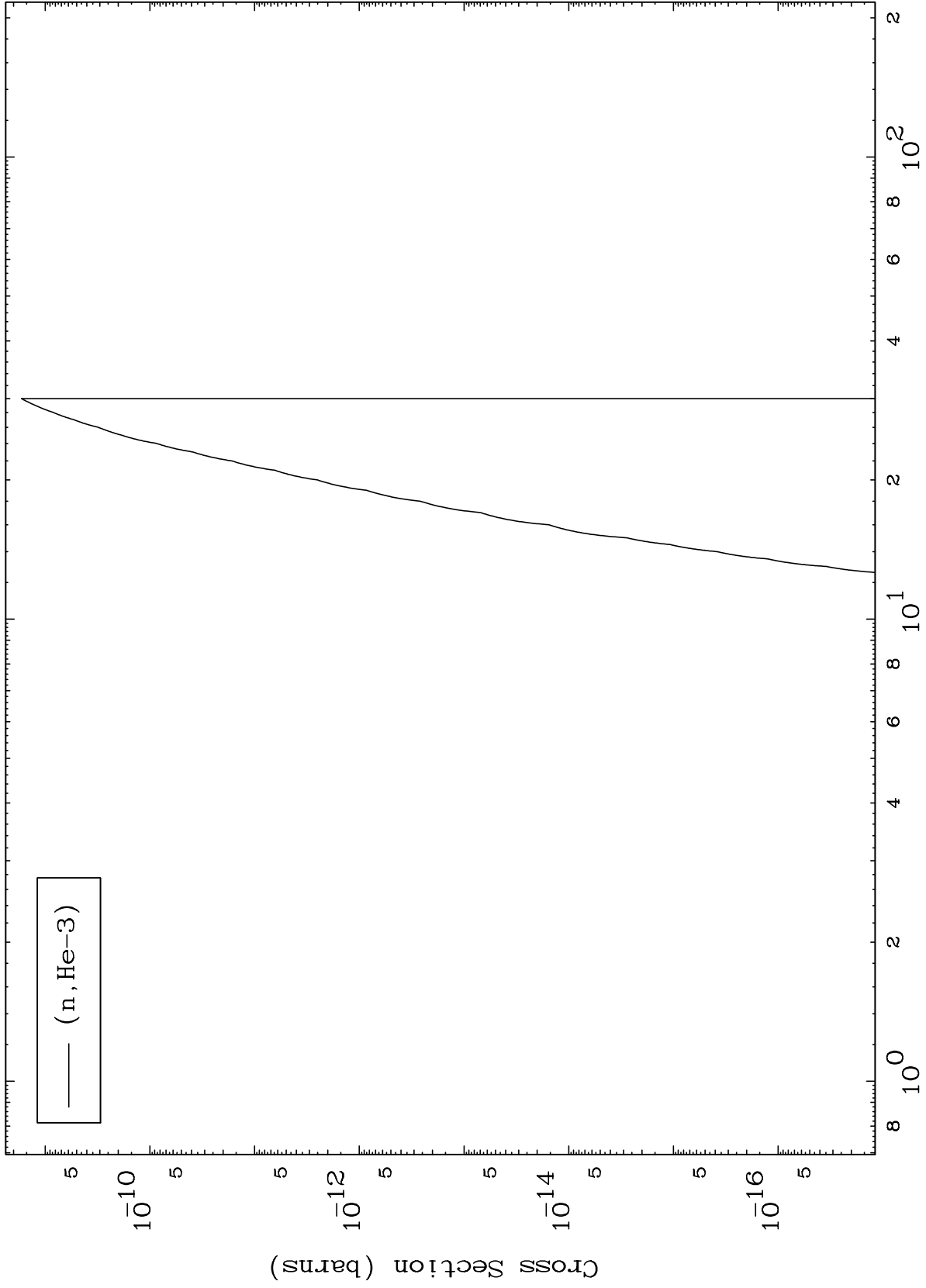
Incident Energy (MeV)

109-Mt-279

MAT 979

(n,He3) Levels  
293 Kelvin Cross Sections

109-Mt-279



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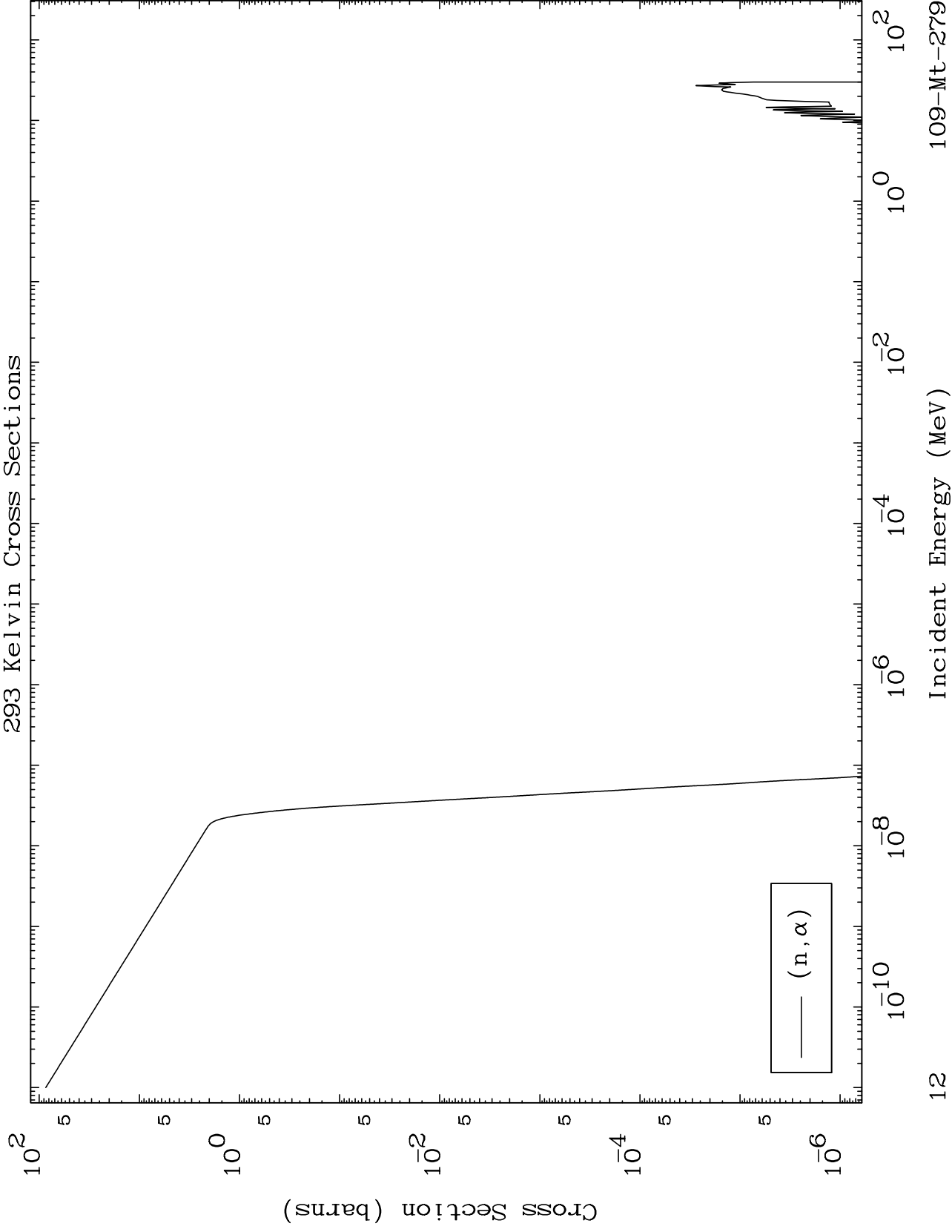
Incident Energy (MeV)

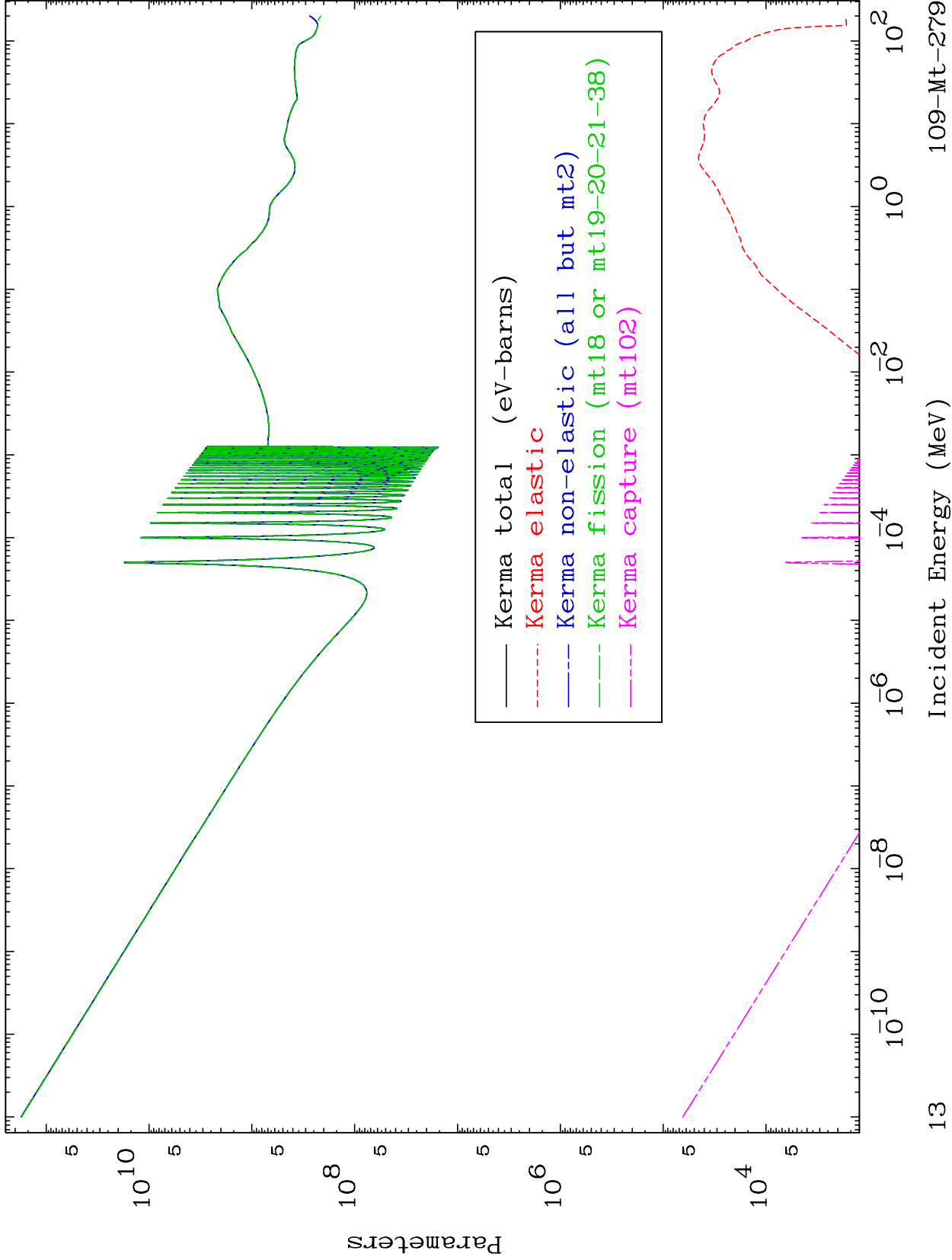
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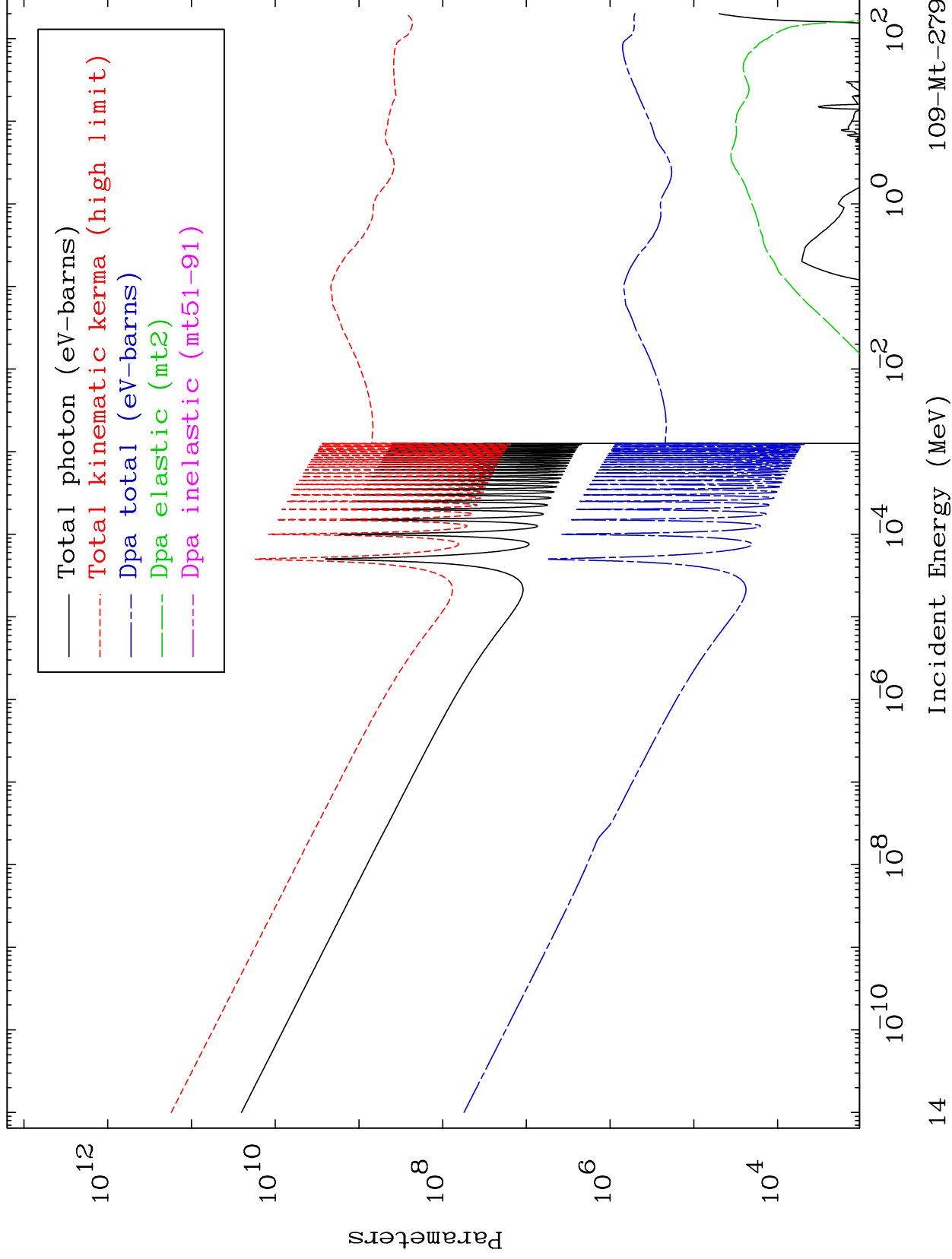
MAT 979

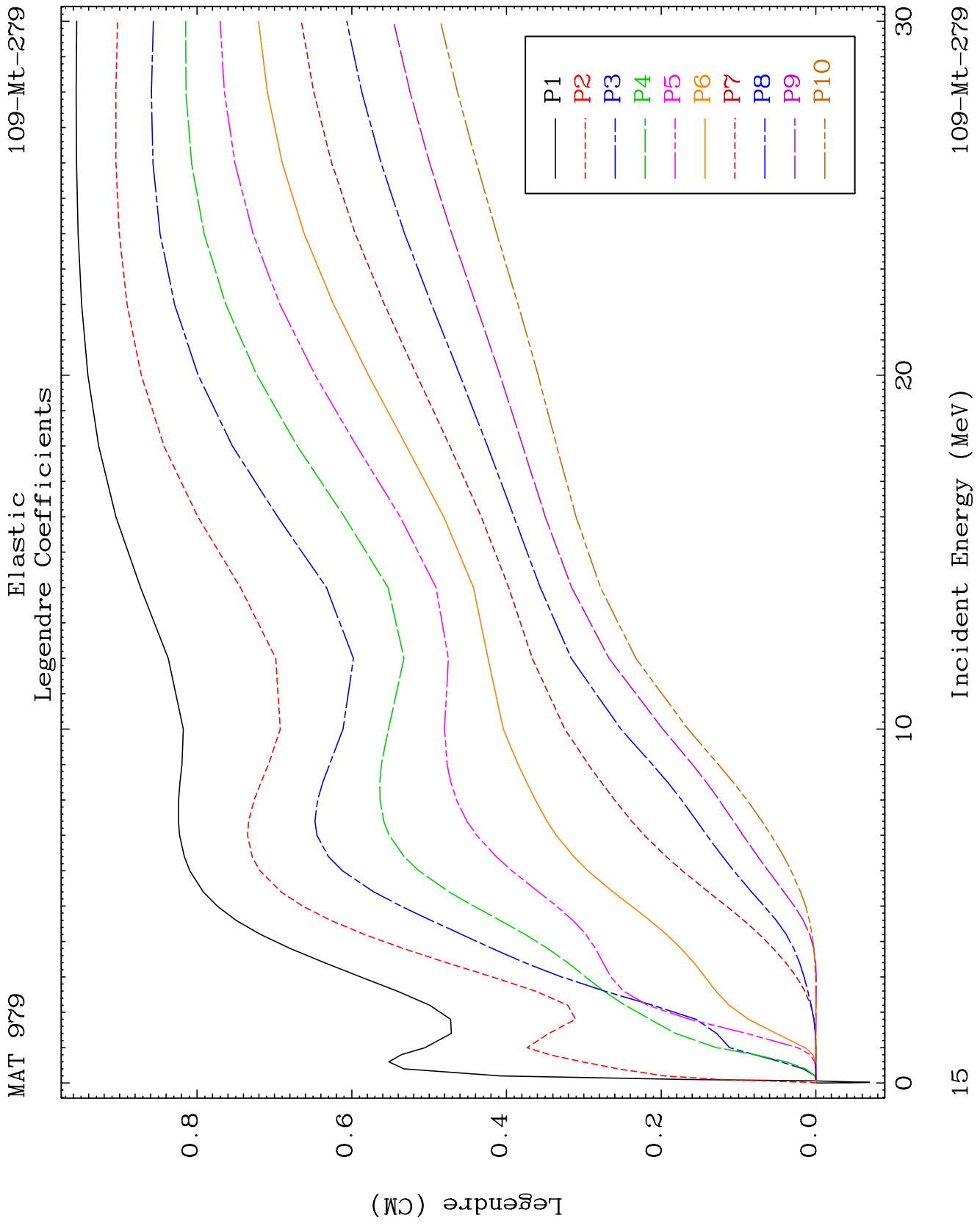
(n,  $\alpha$ ) Levels  
293 Kelvin Cross Sections

109-Mt-279

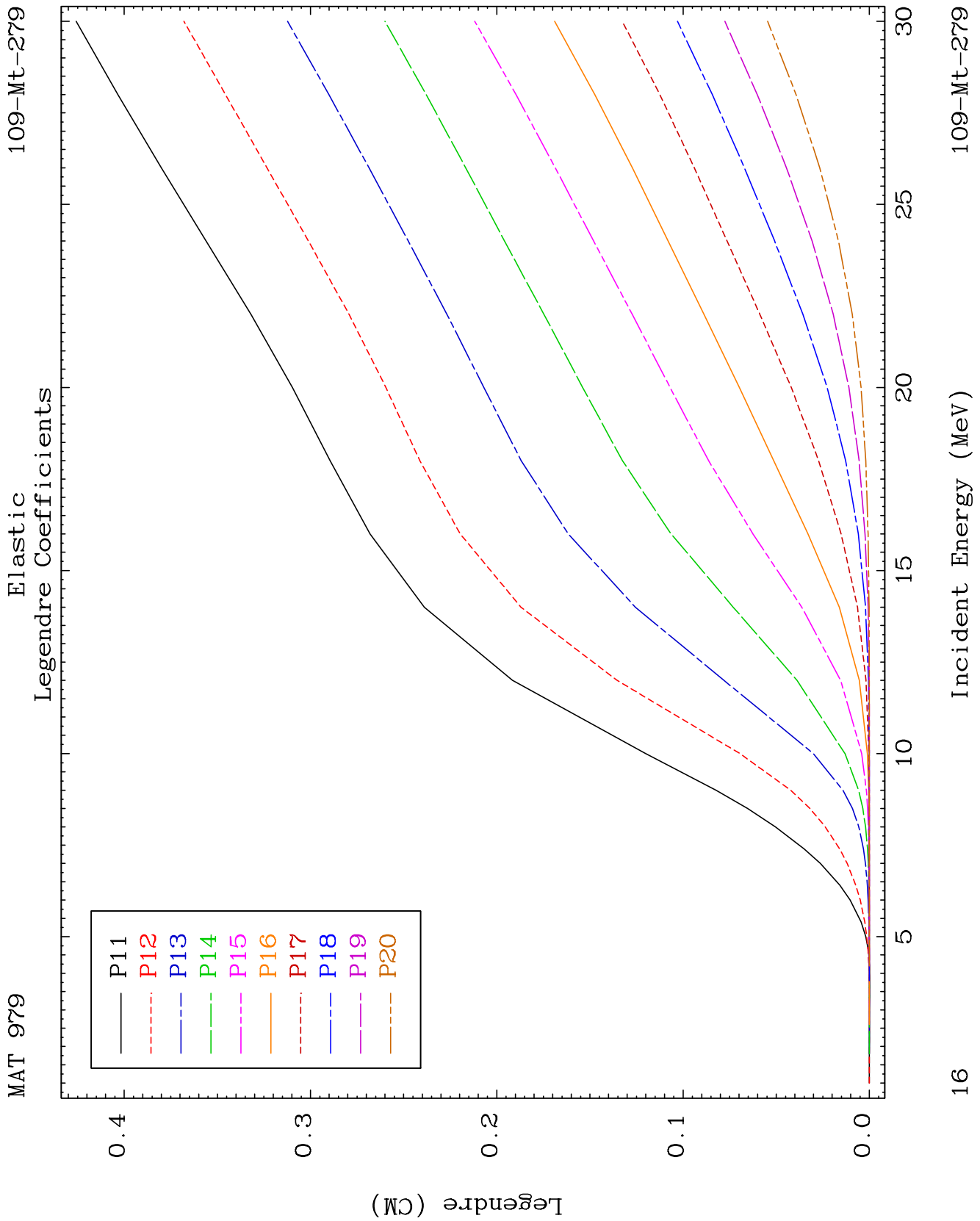


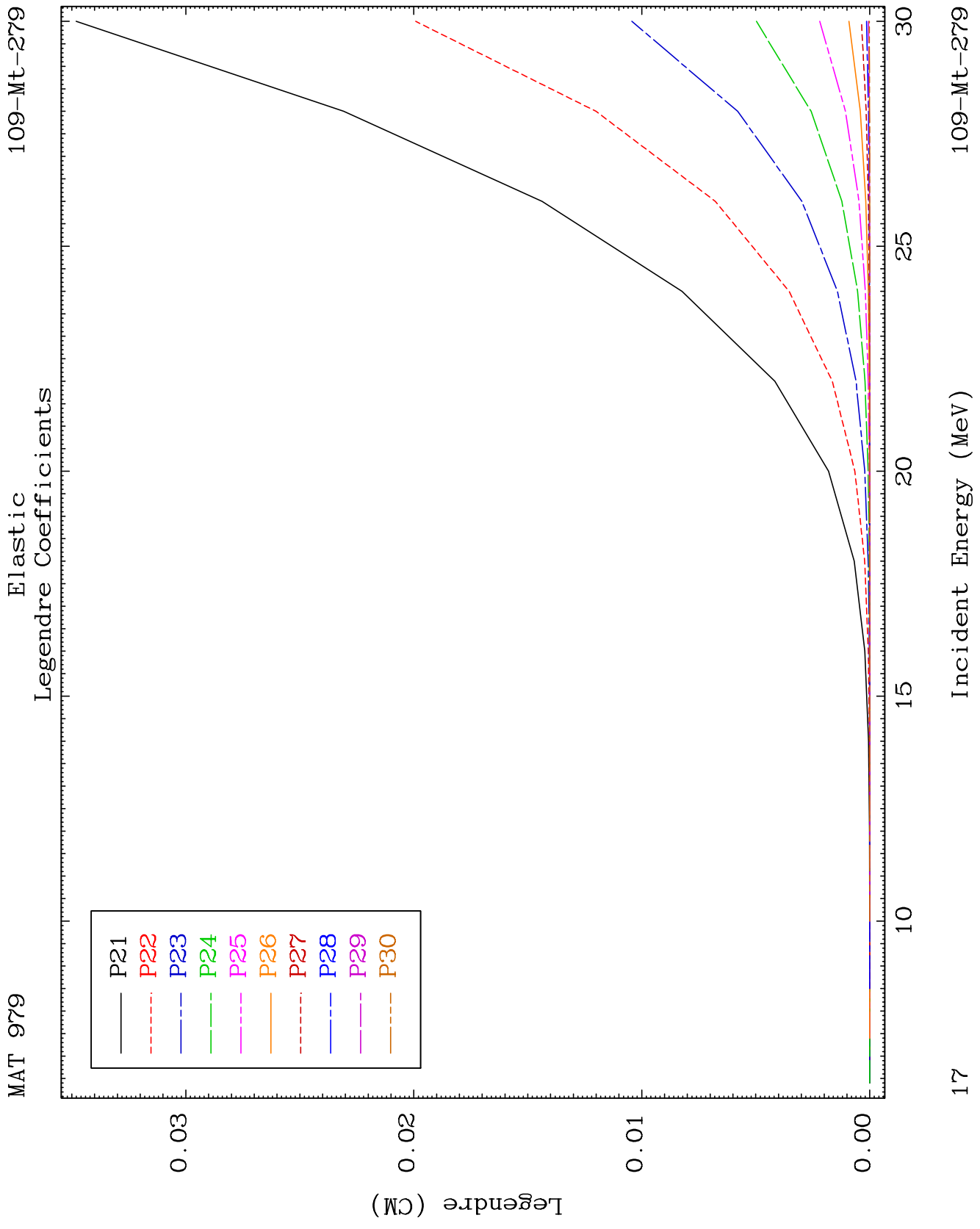










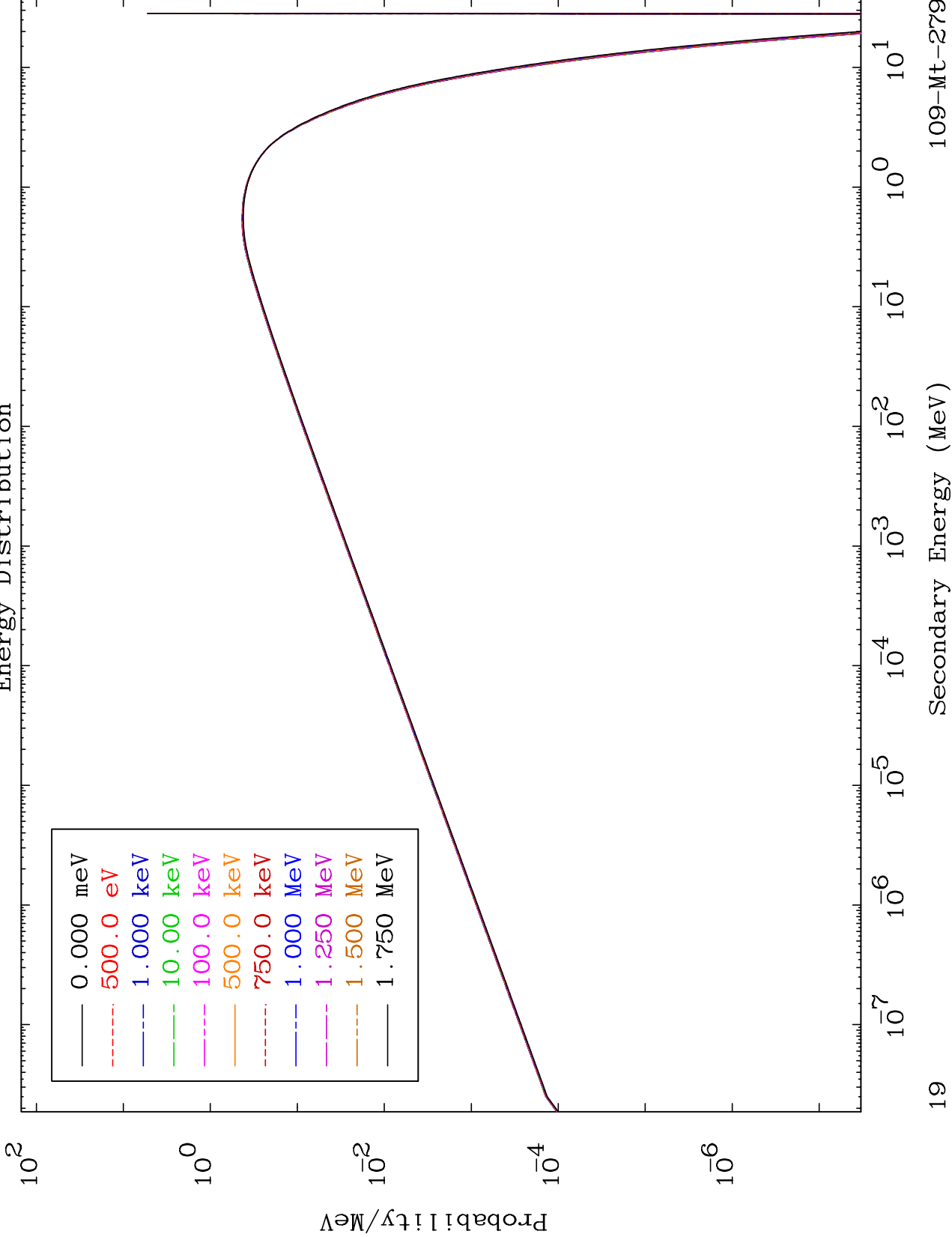




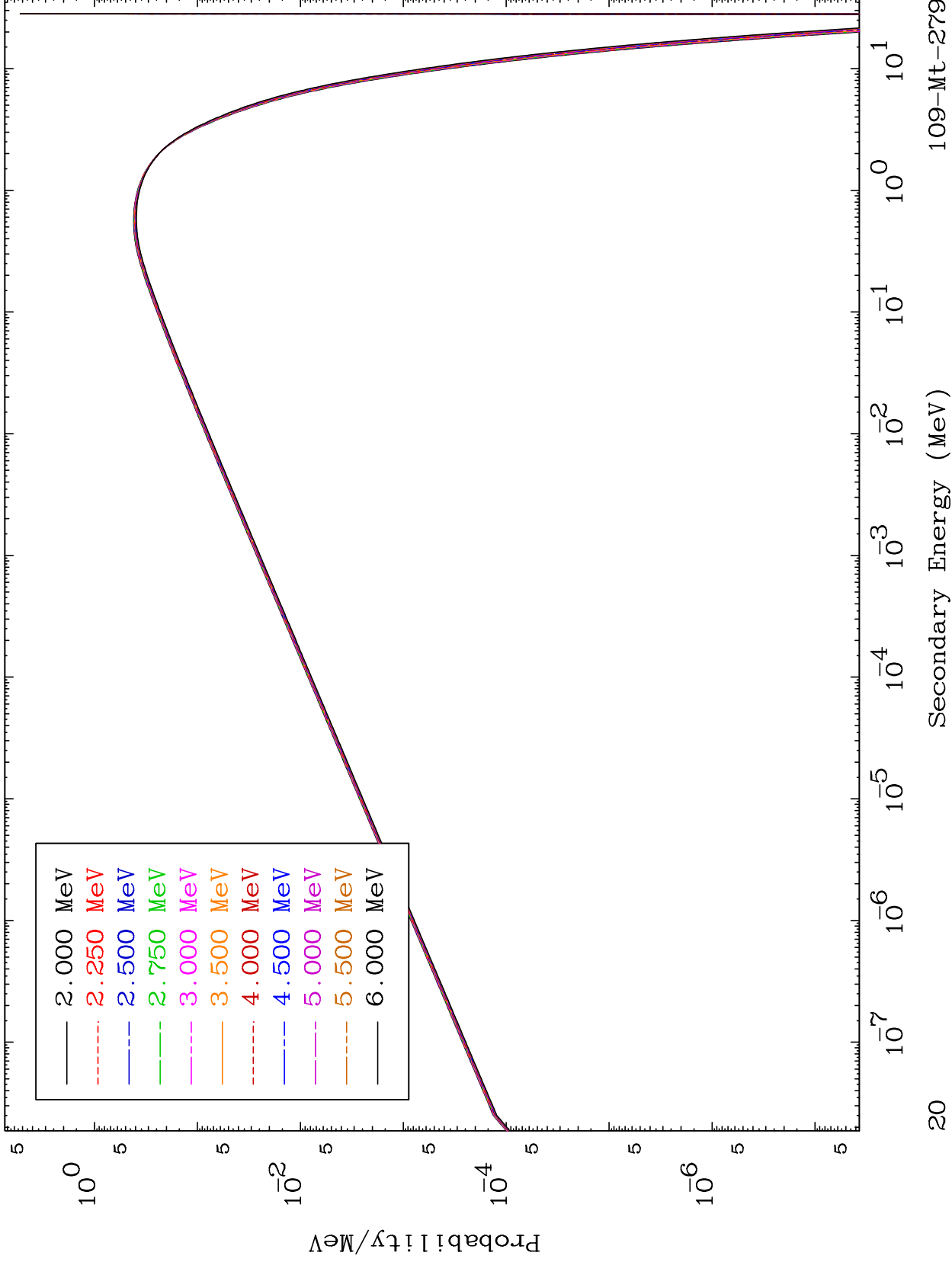
MAT 979

### Fission Energy Distribution

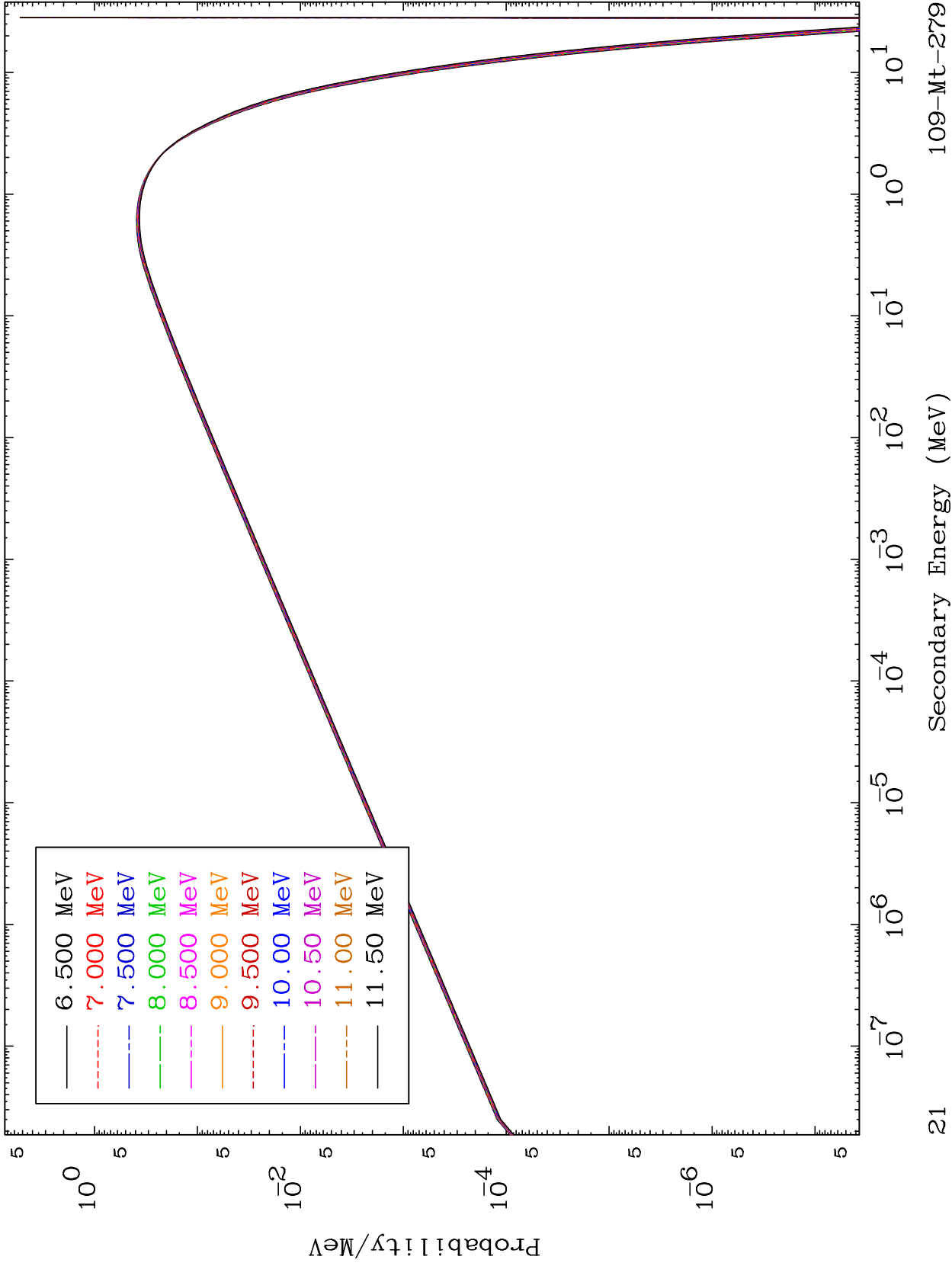
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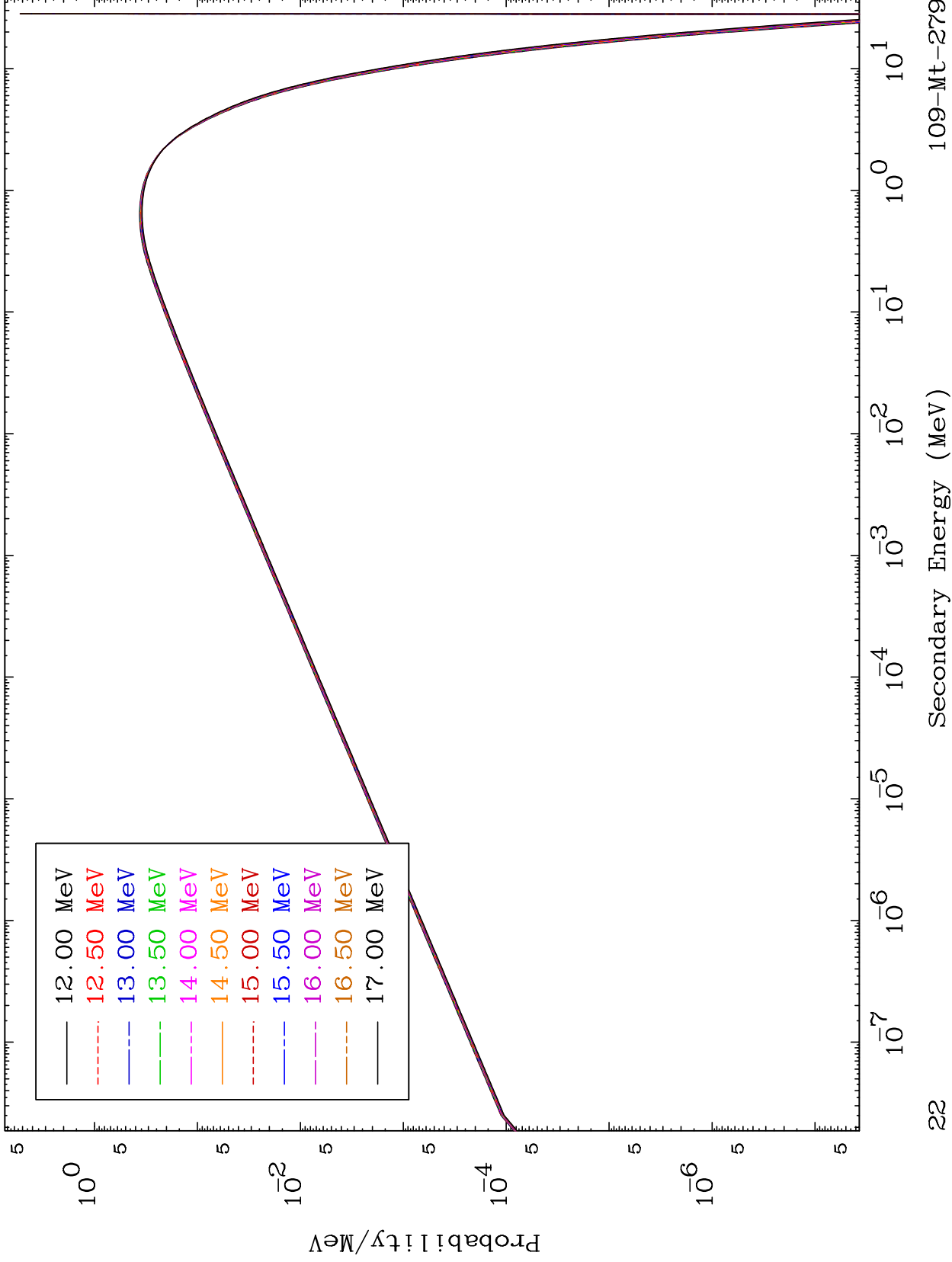


Fission Energy Distribution



Fission  
Energy Distribution





Fission  
Energy Distribution

