

Program EVALPLOT
(Version 2015-2)

by

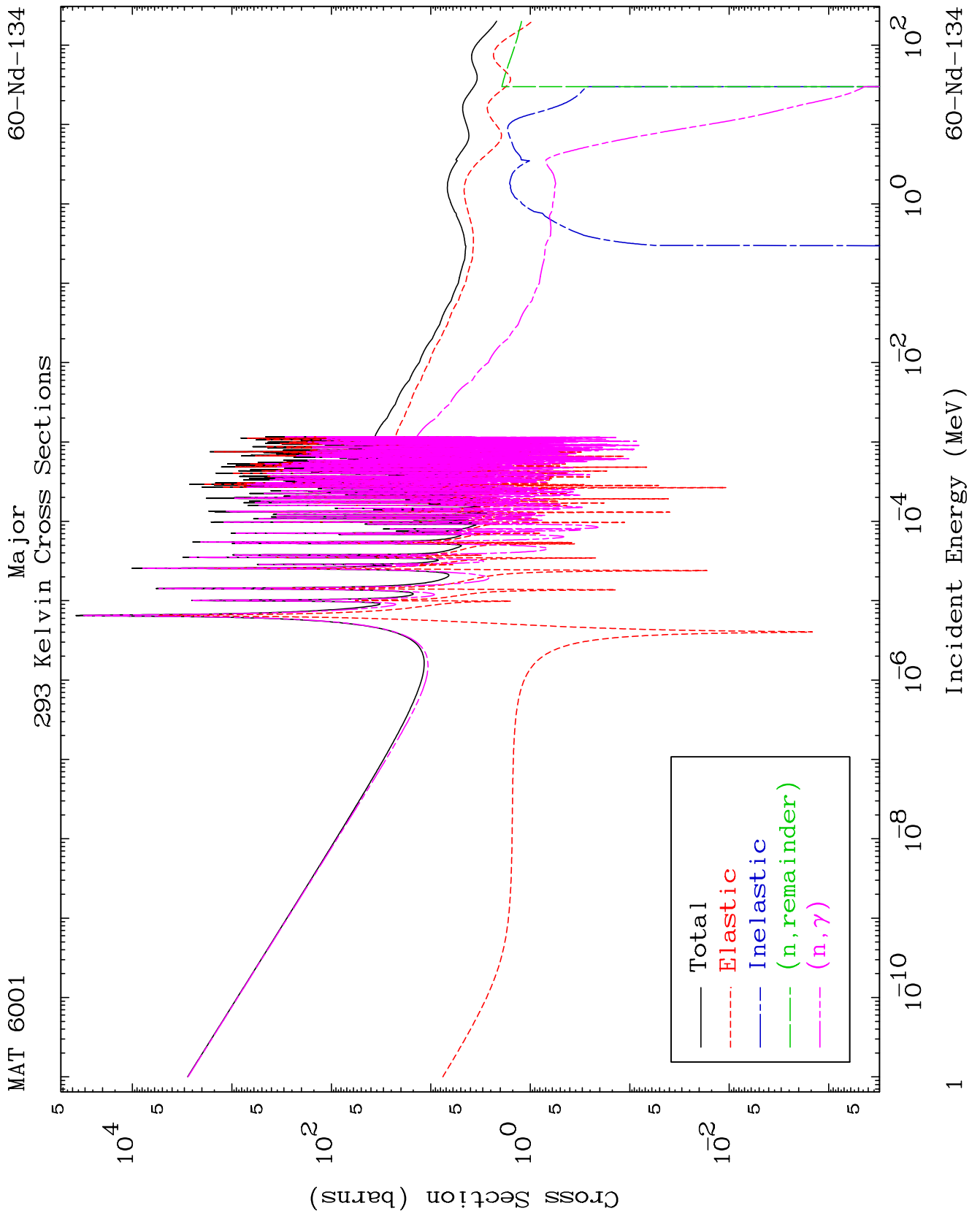
Dermott E. Cullen
(Present Contact Information)

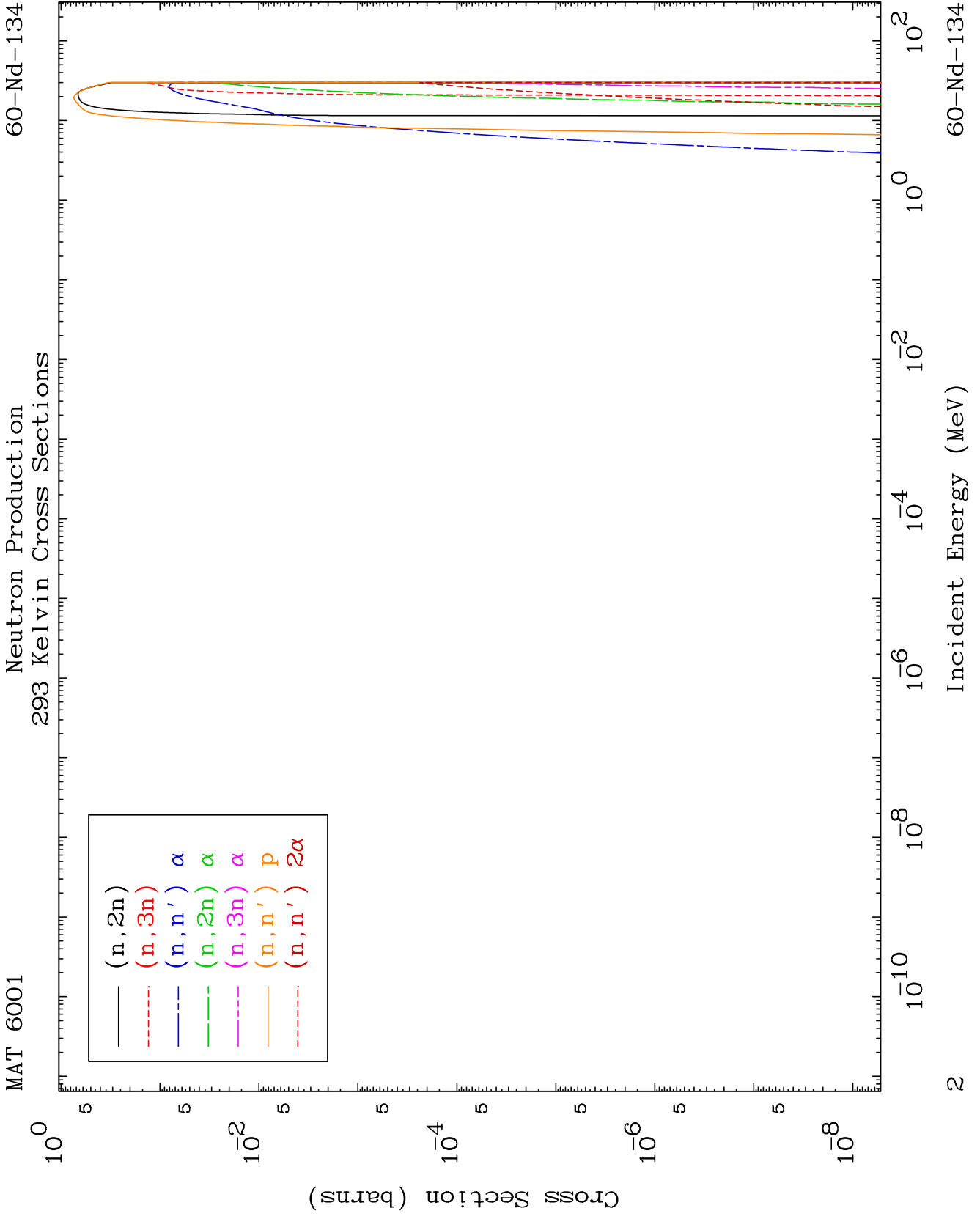
Dermott E. Cullen
1466 Hudson Way
Livermore, CA 94550
U.S.A.

Tele: 925-443-1911

E.Mail:redcullen1@comcast.net
Web:home.comcast.net/~redcullen1

Press Mouse Button to Start

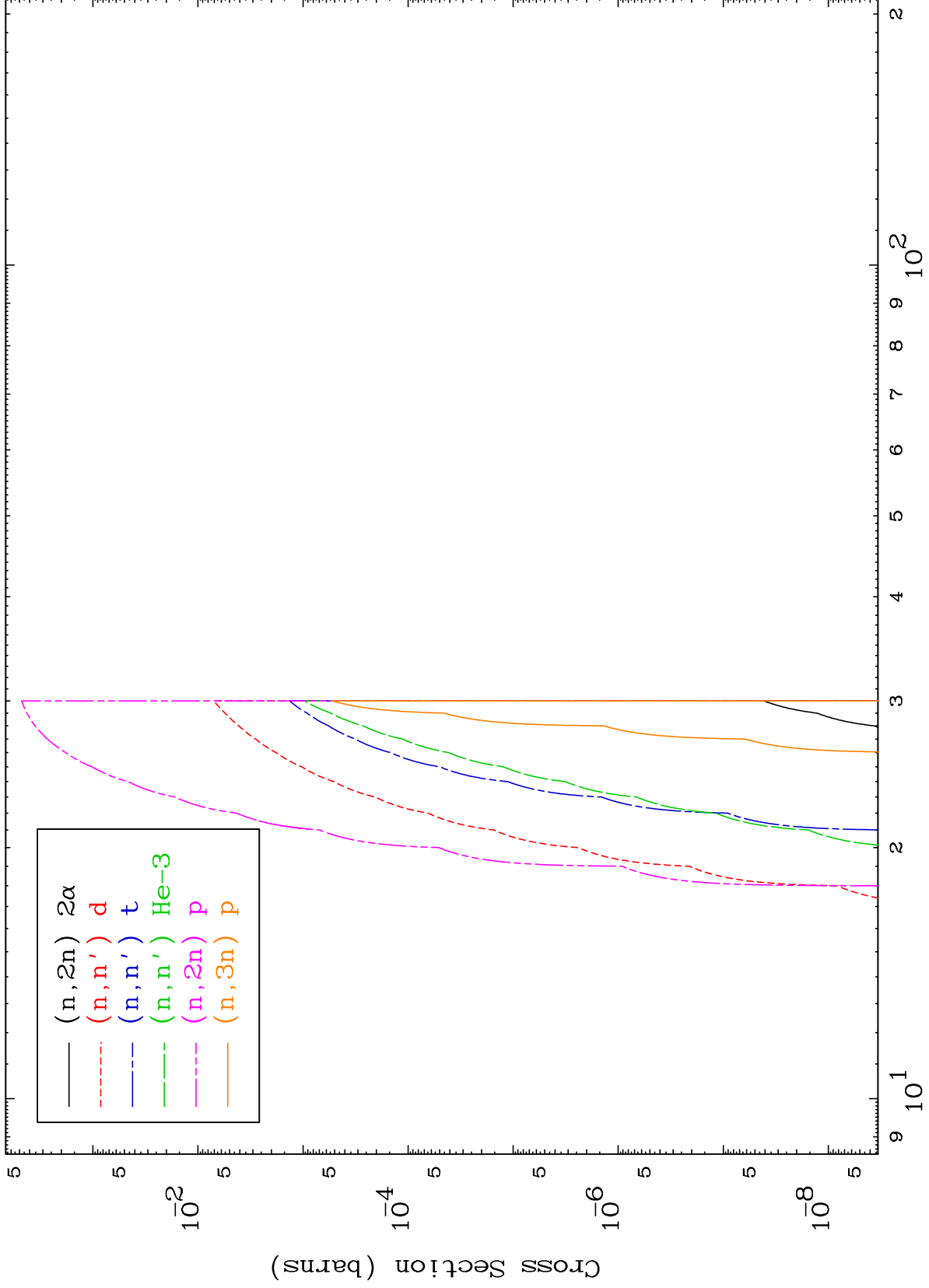




MAT 6001

Neutron Production
293 Kelvin Cross Sections

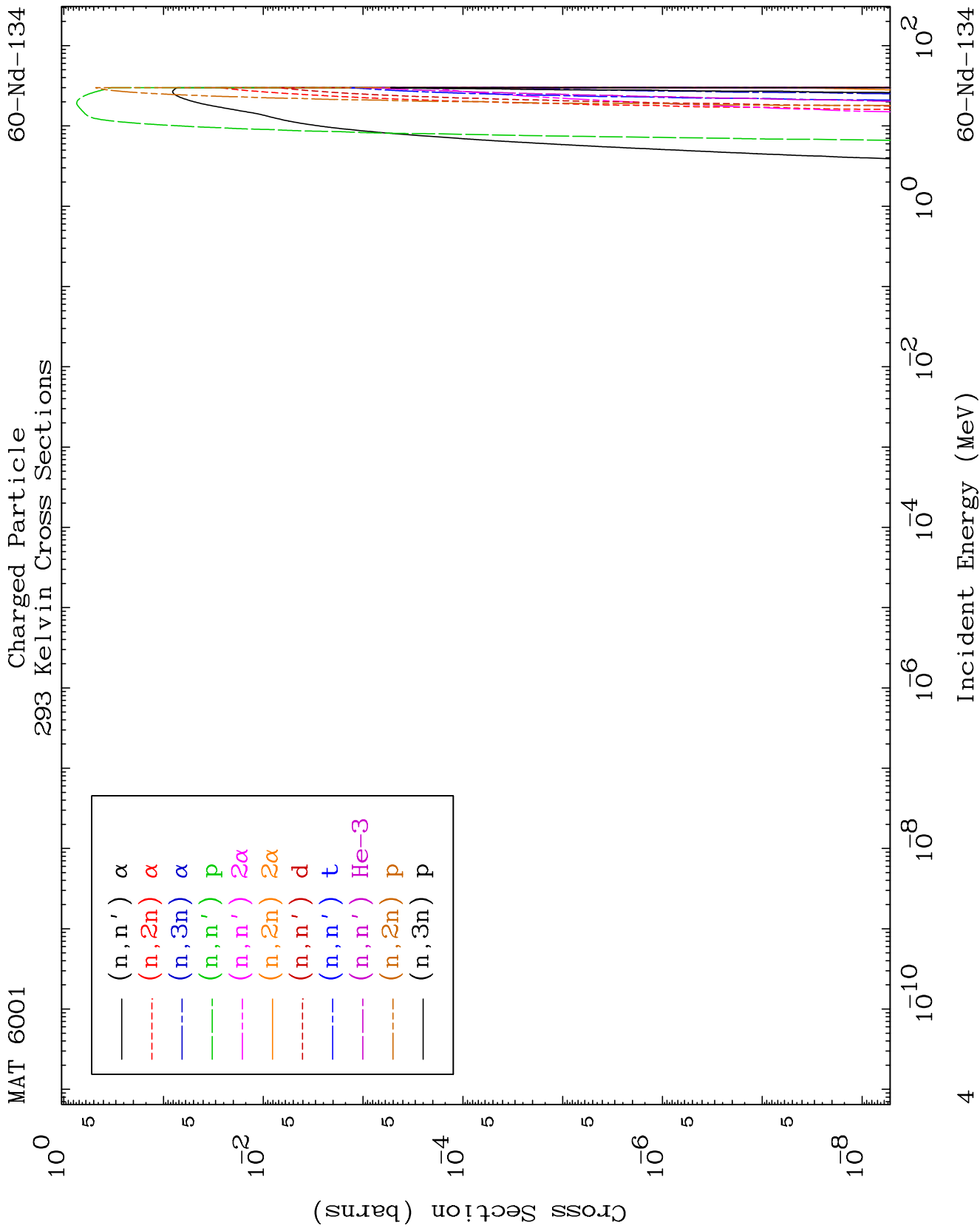
60-Nd-134



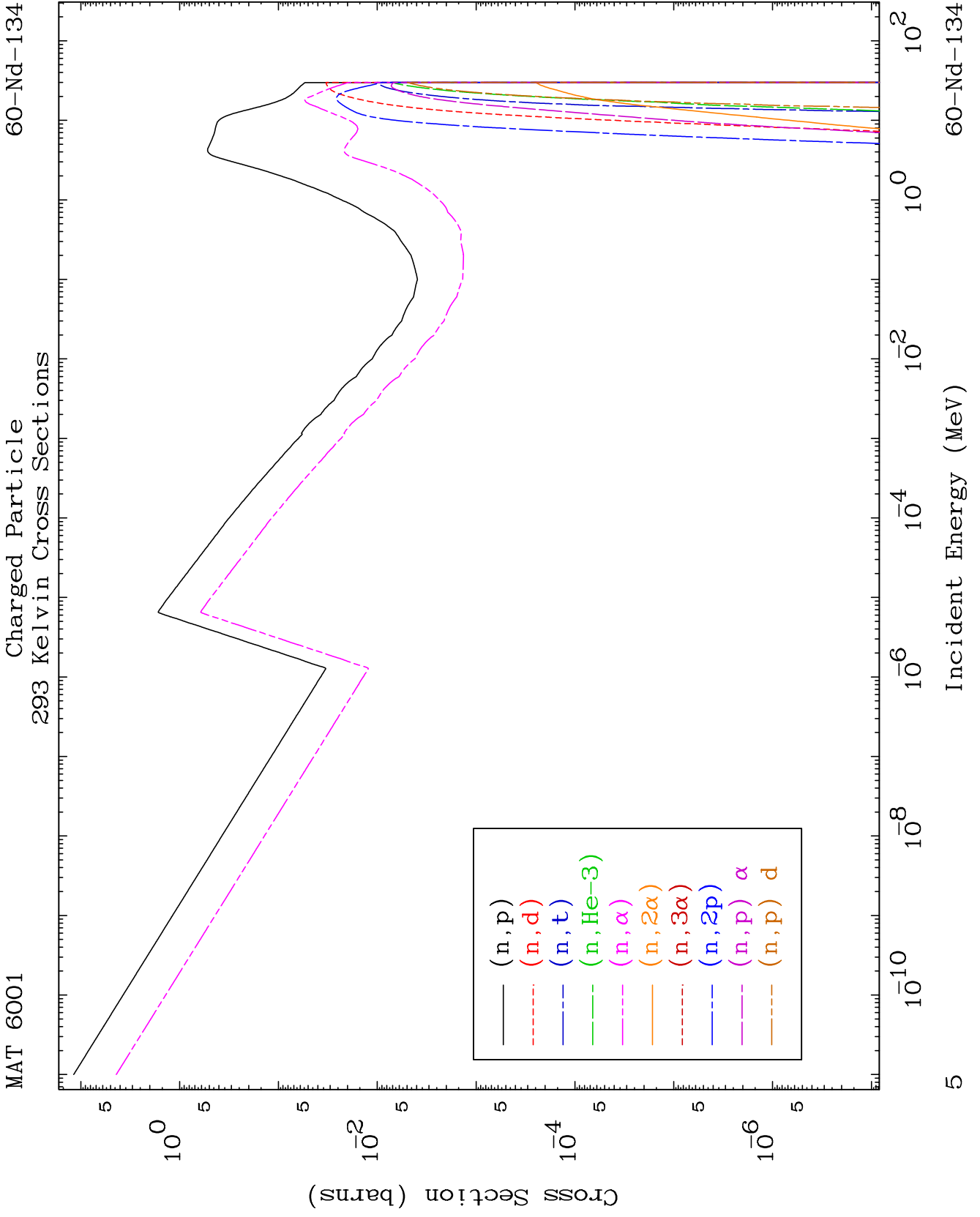
Incident Energy (MeV)

60-Nd-134

3



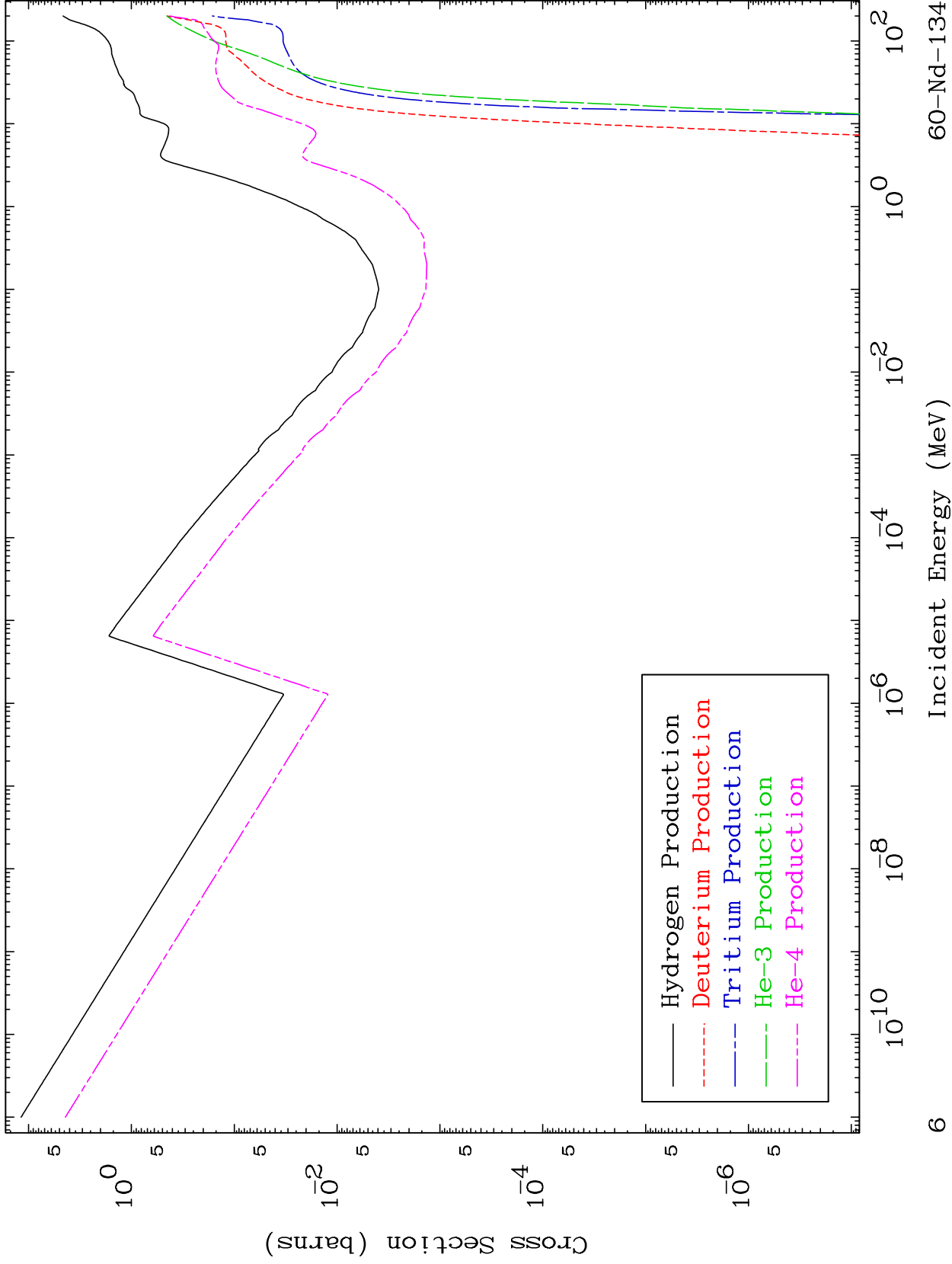
MAT 6001



MAT 6001

Particle Production
293 Kelvin Cross Sections

60-Nd-134

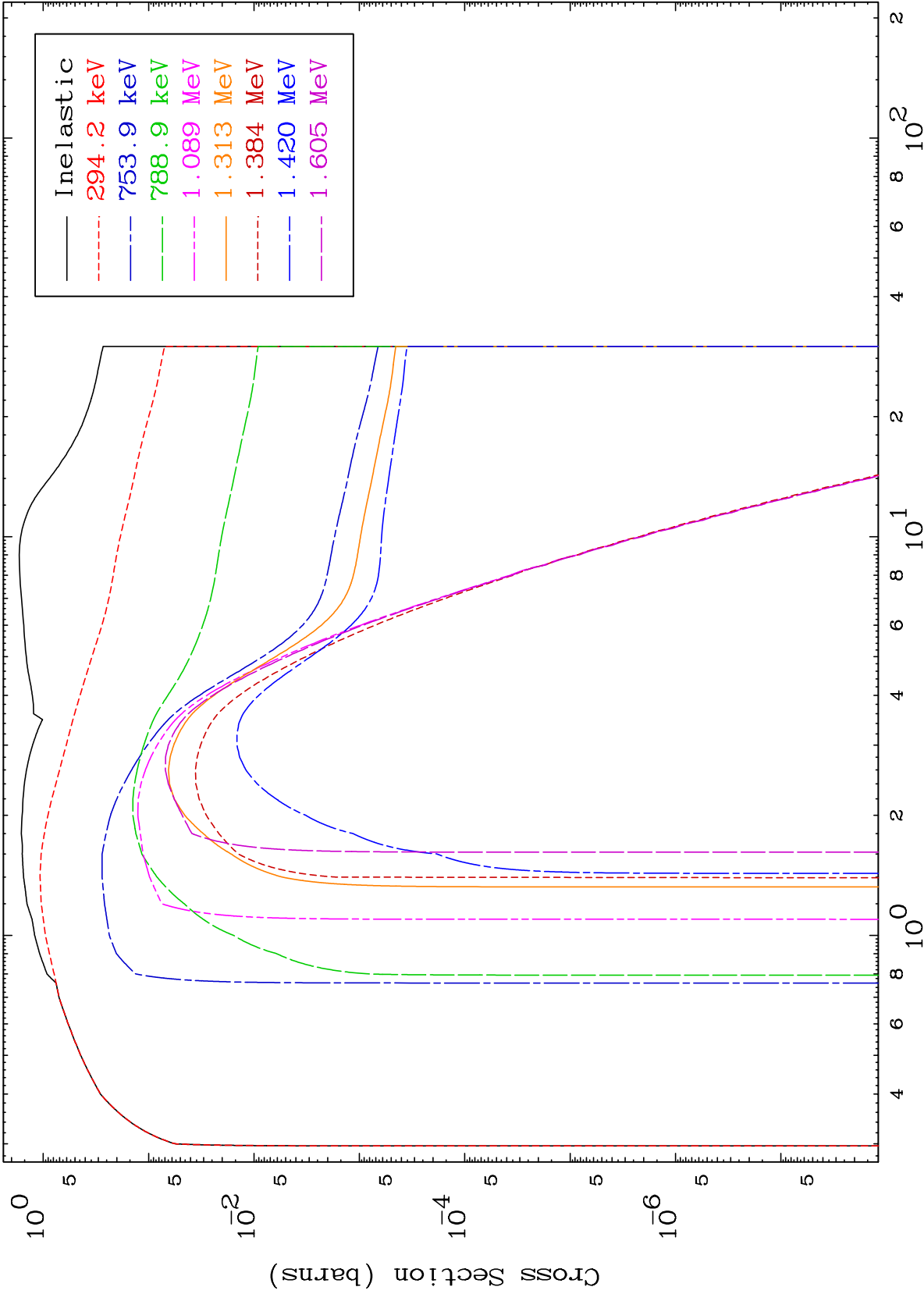


MAT 6001

(n,n') Level

60-Nd-134

293 Kelvin Cross Sections

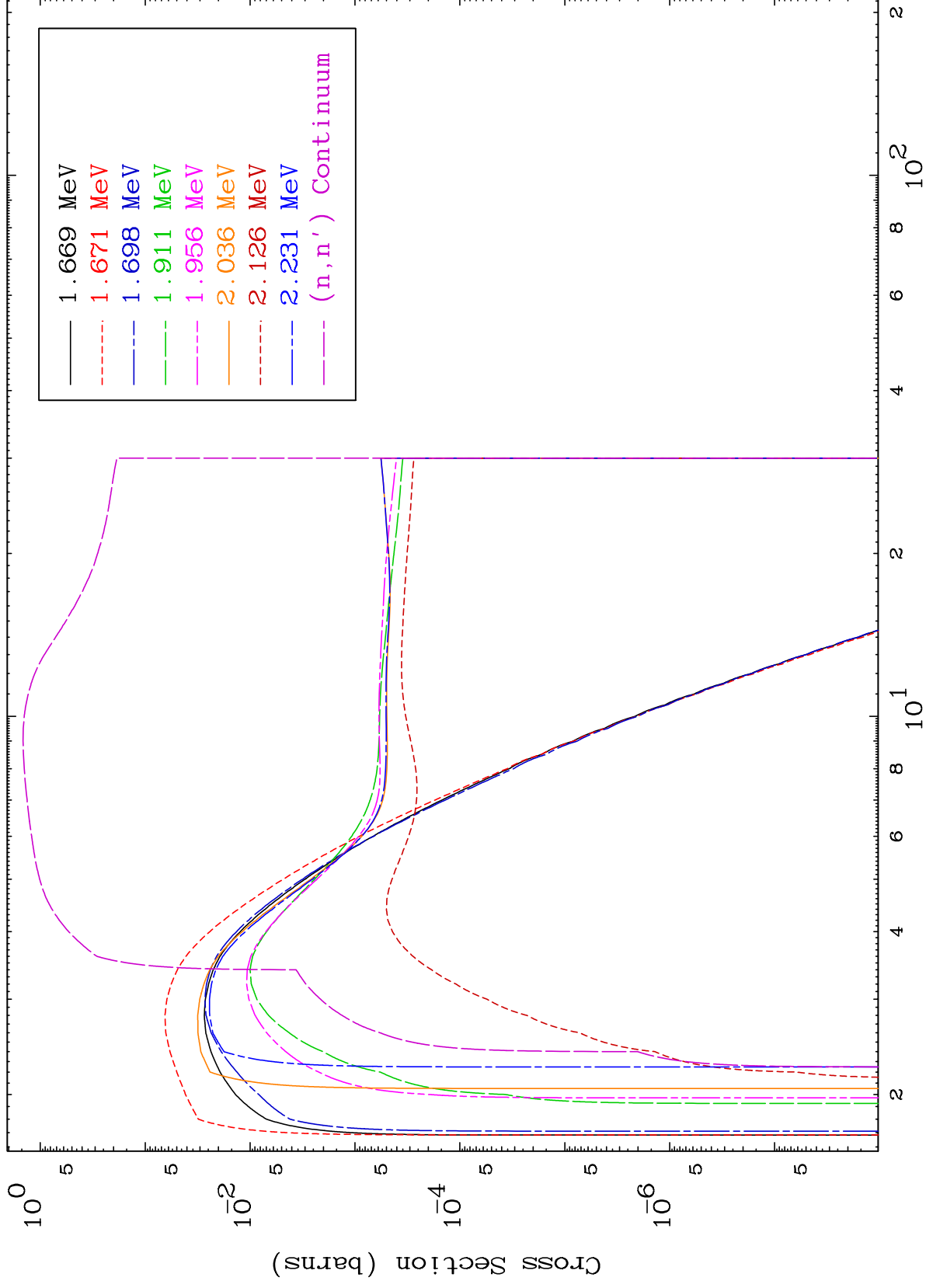


MAT 6001

(n,n') Level

60-Nd-134

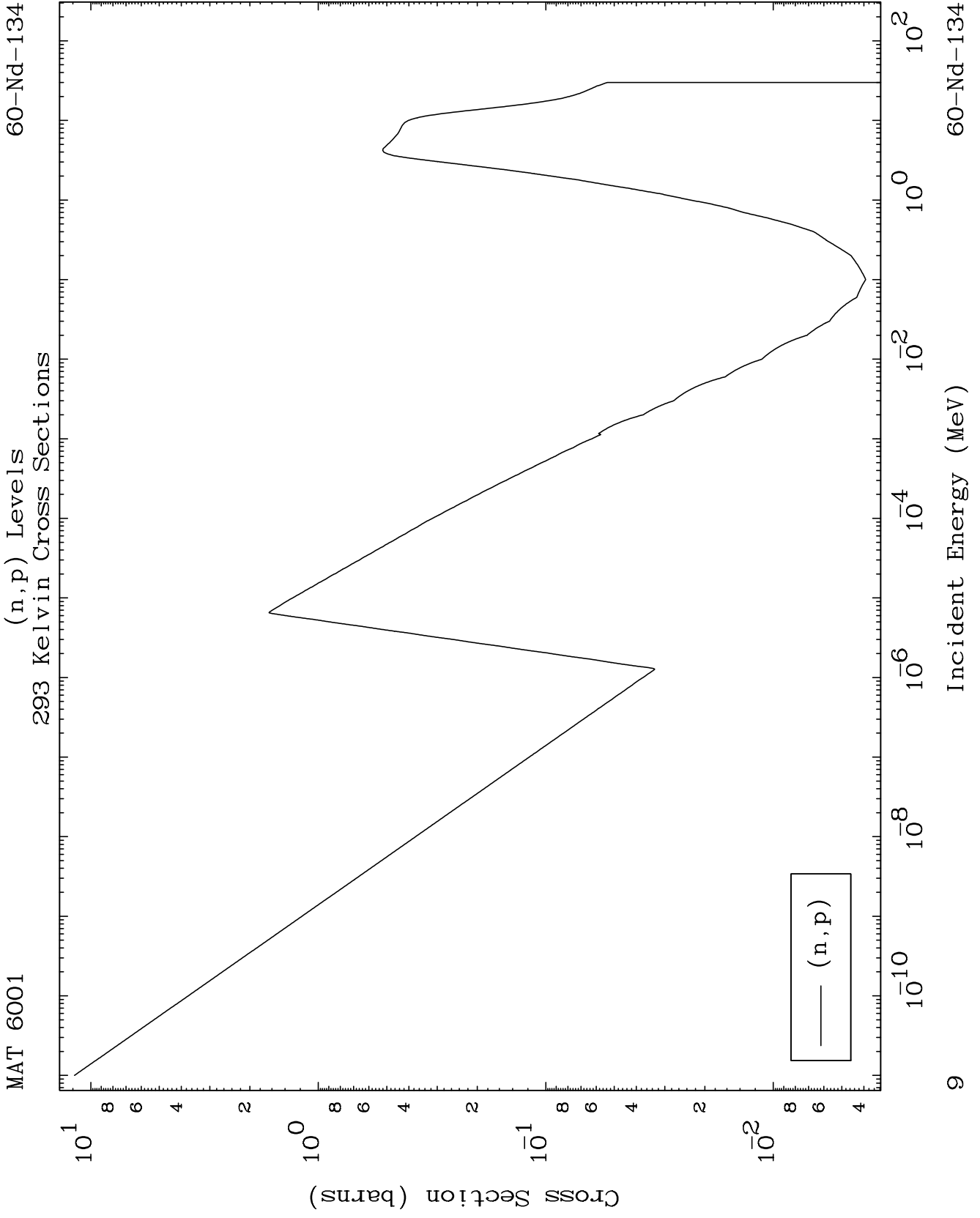
293 Kelvin Cross Sections



8

Incident Energy (MeV)

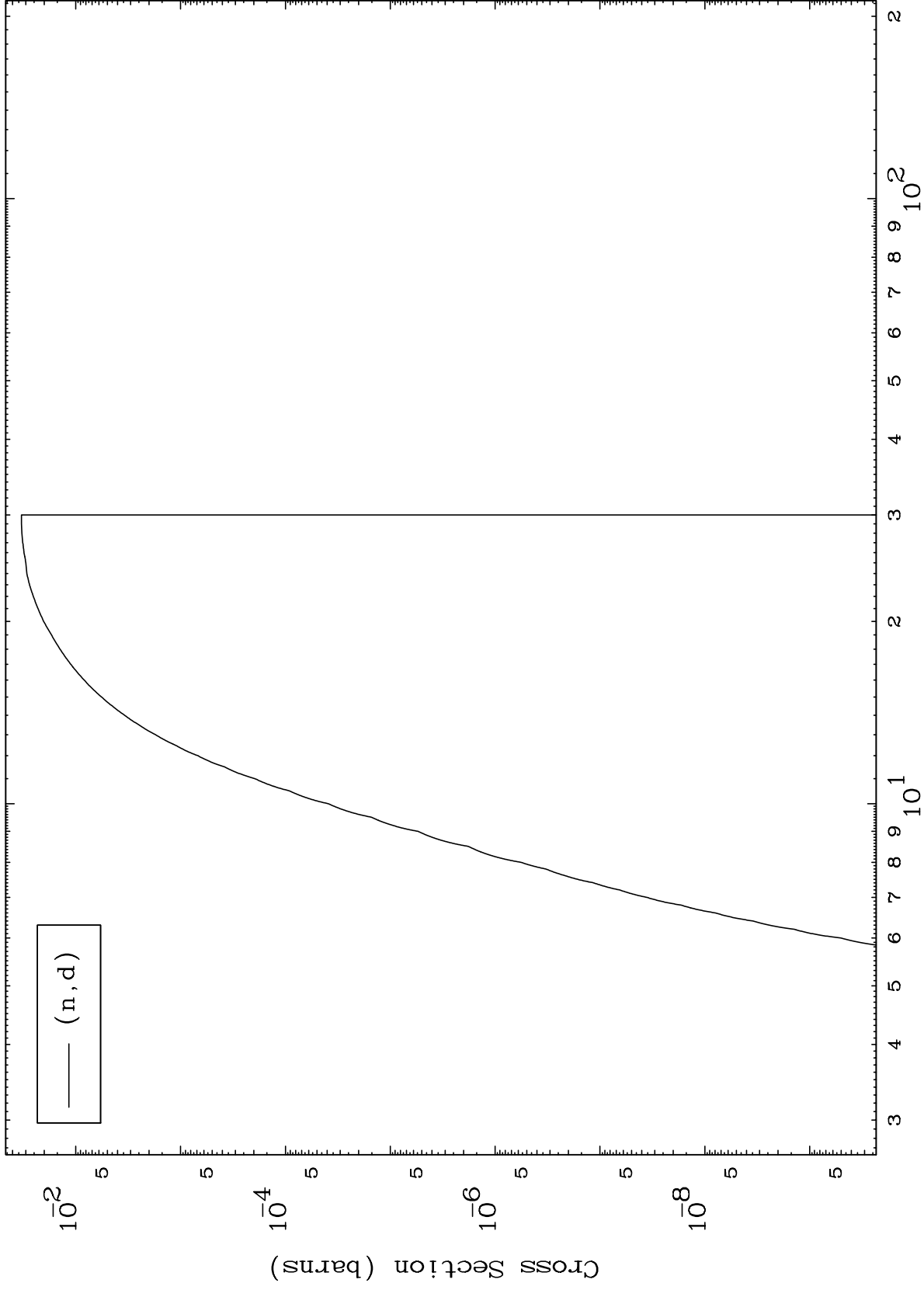
60-Nd-134



MAT 6001

(n,d) Levels
293 Kelvin Cross Sections

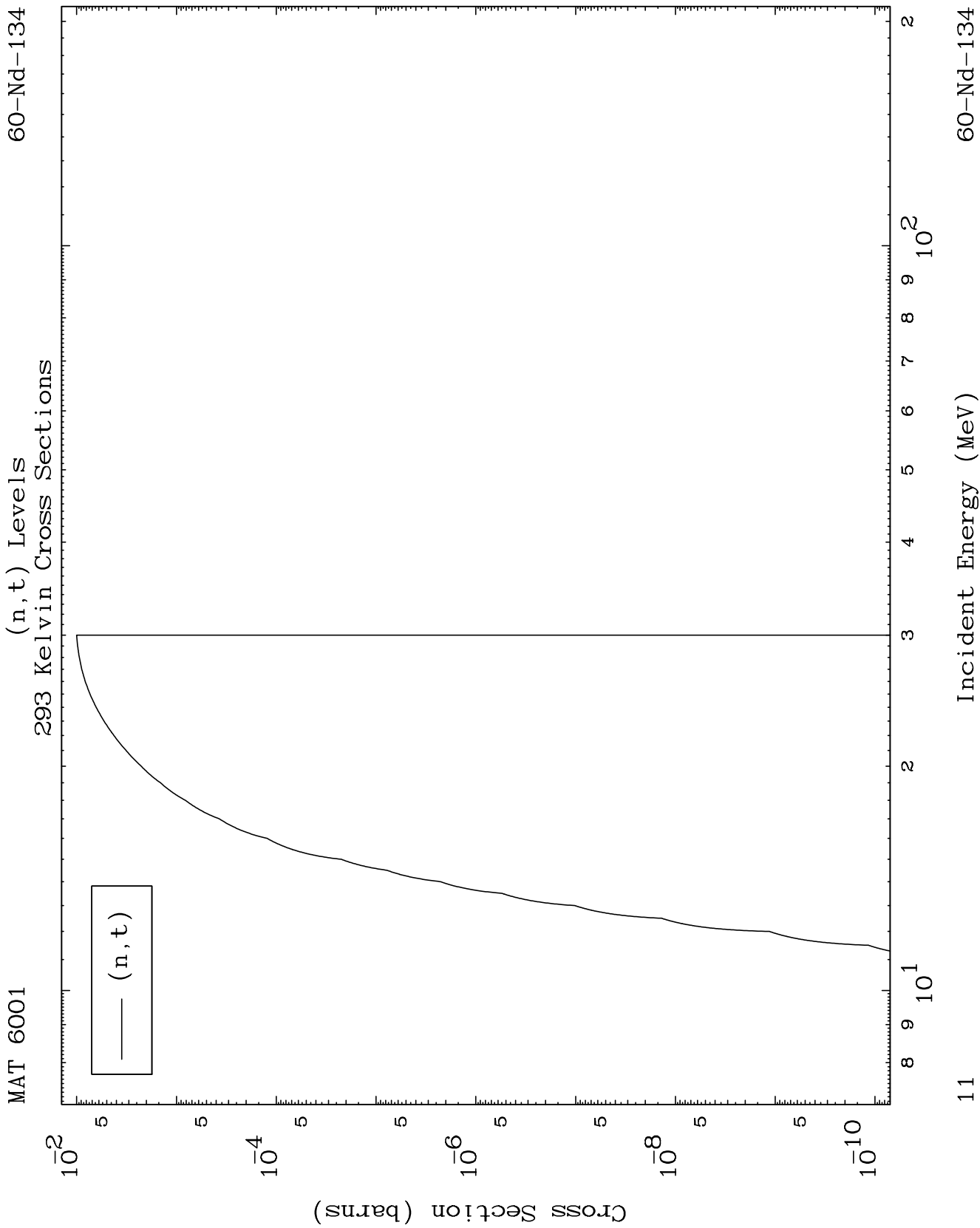
60-Nd-134

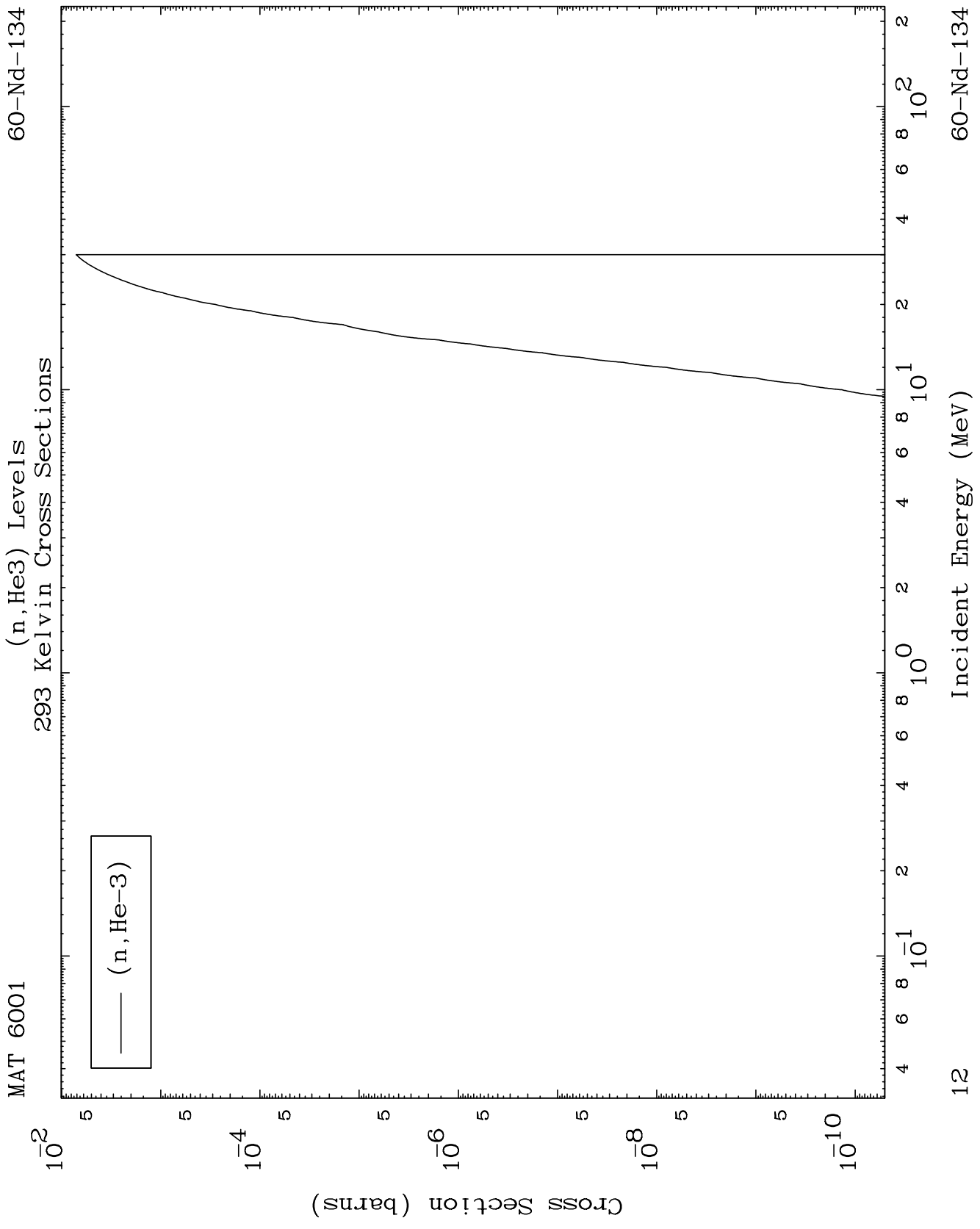


10

Incident Energy (MeV)

60-Nd-134

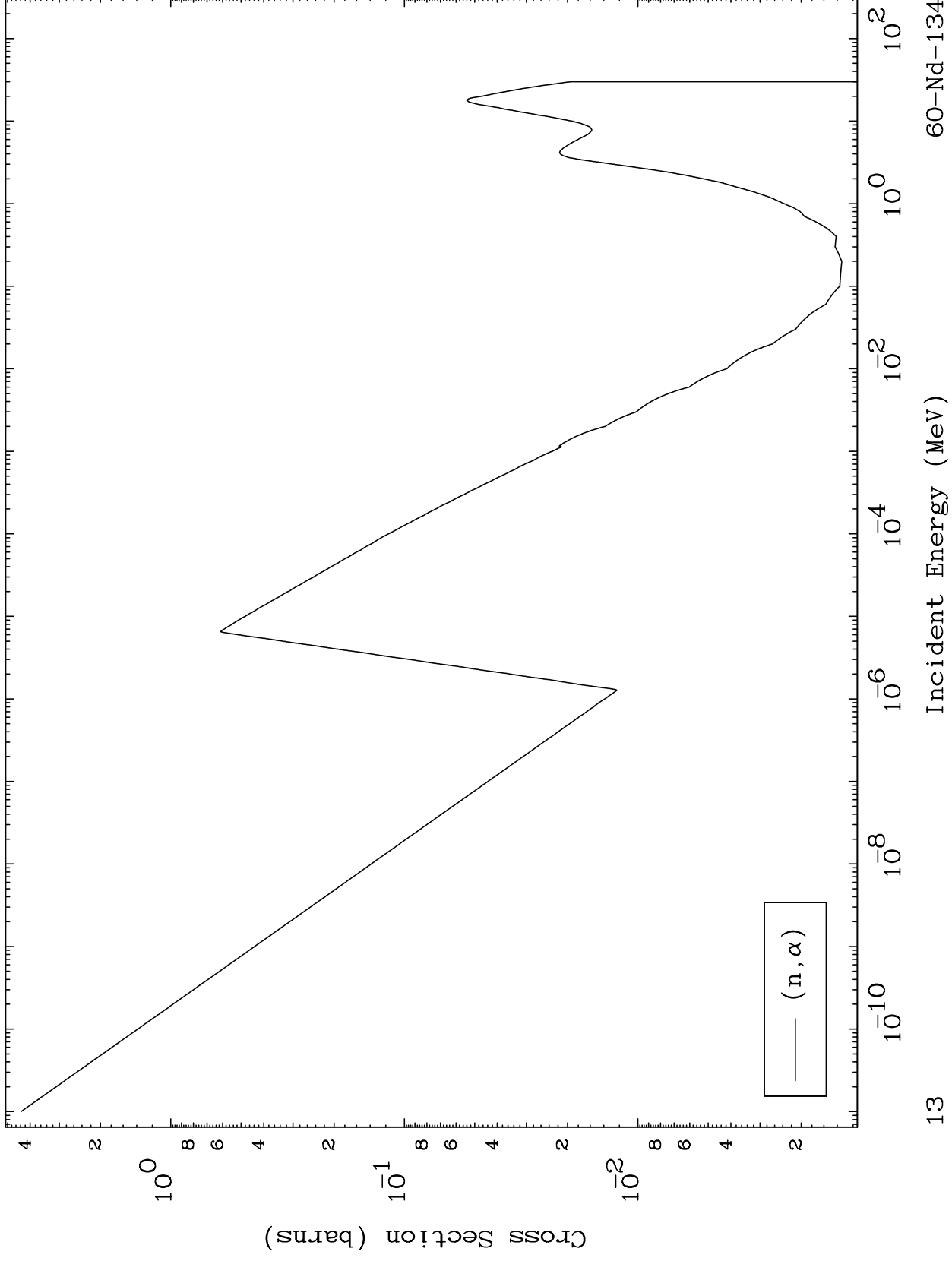


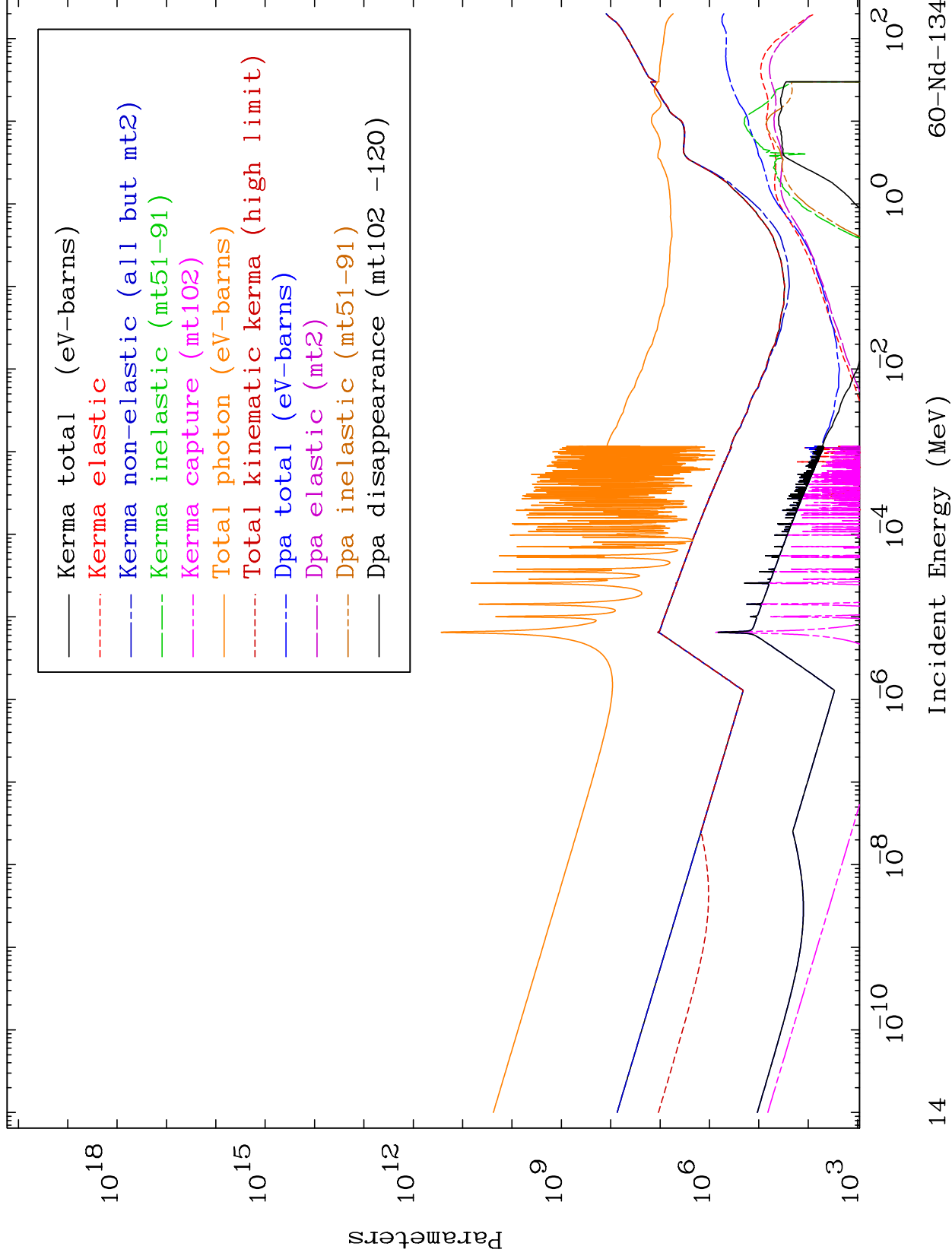


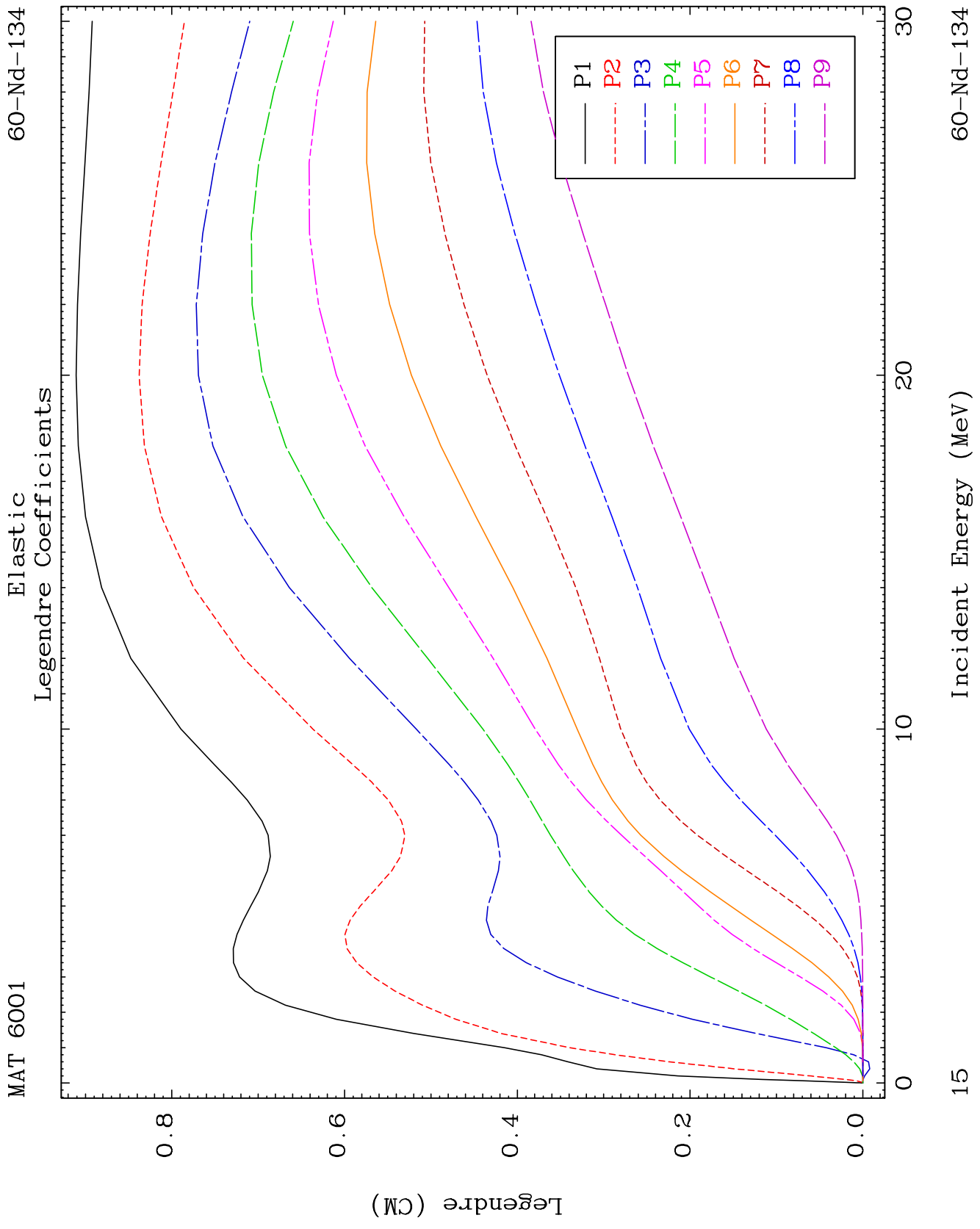
MAT 6001

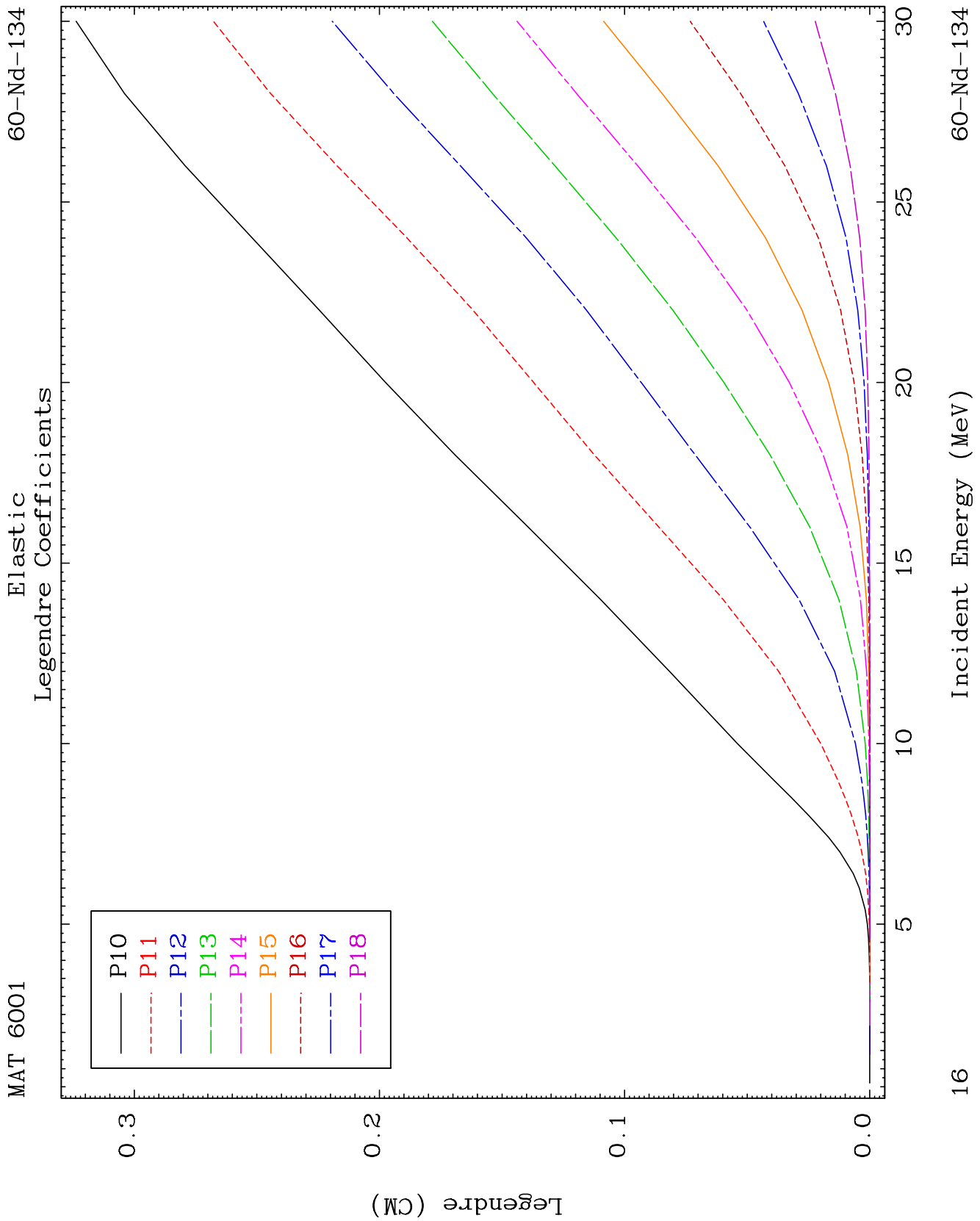
(n, α) Levels
293 Kelvin Cross Sections

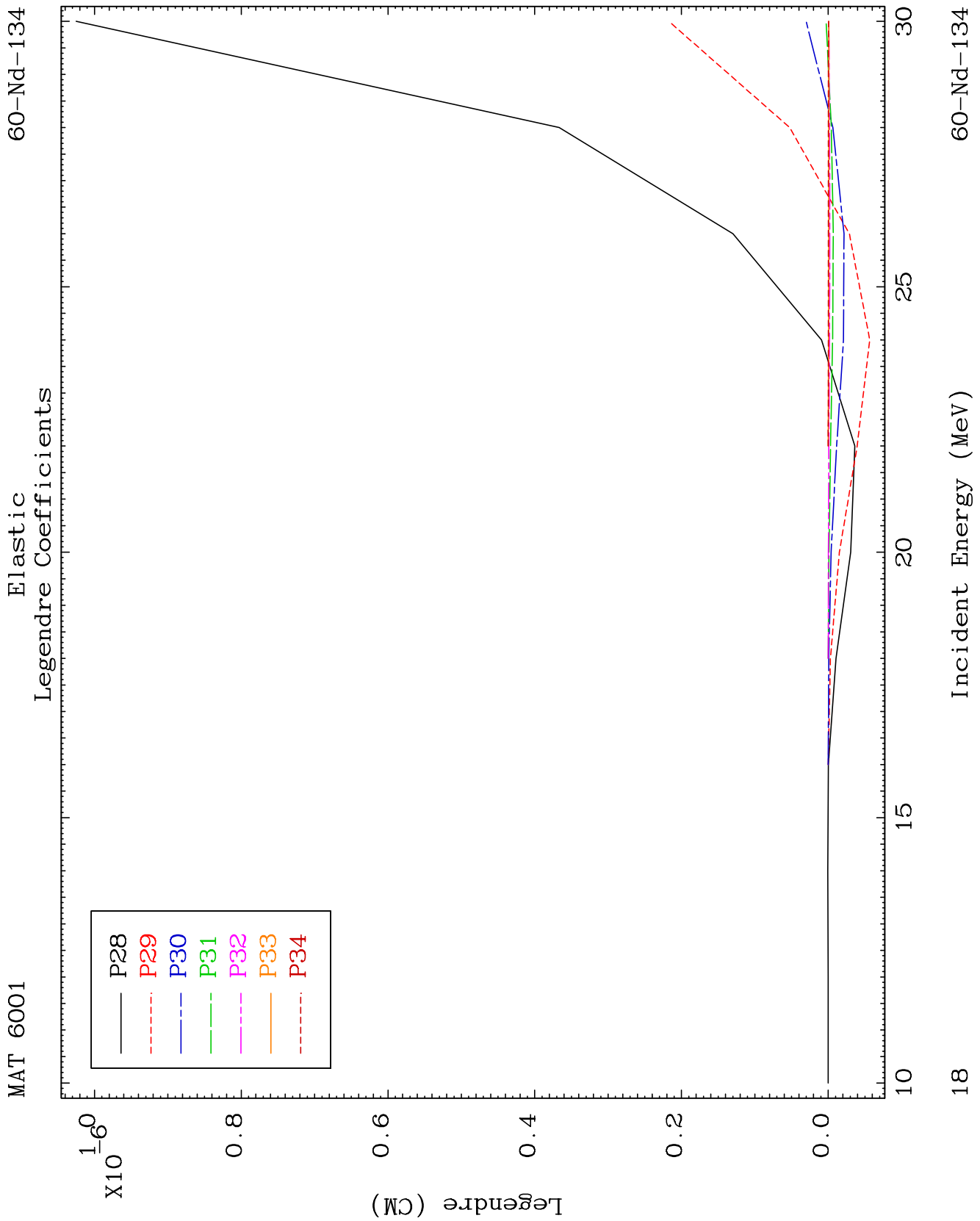
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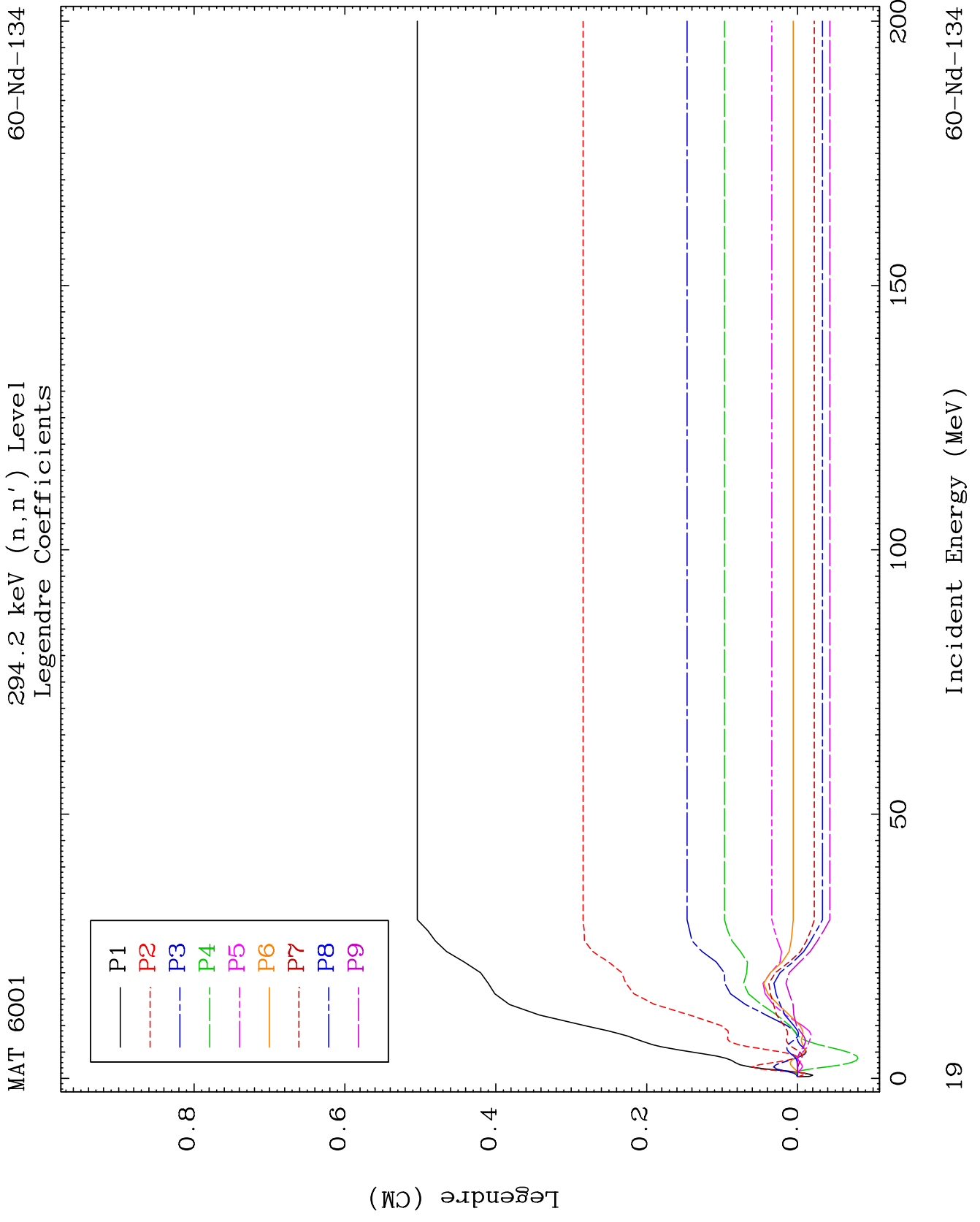


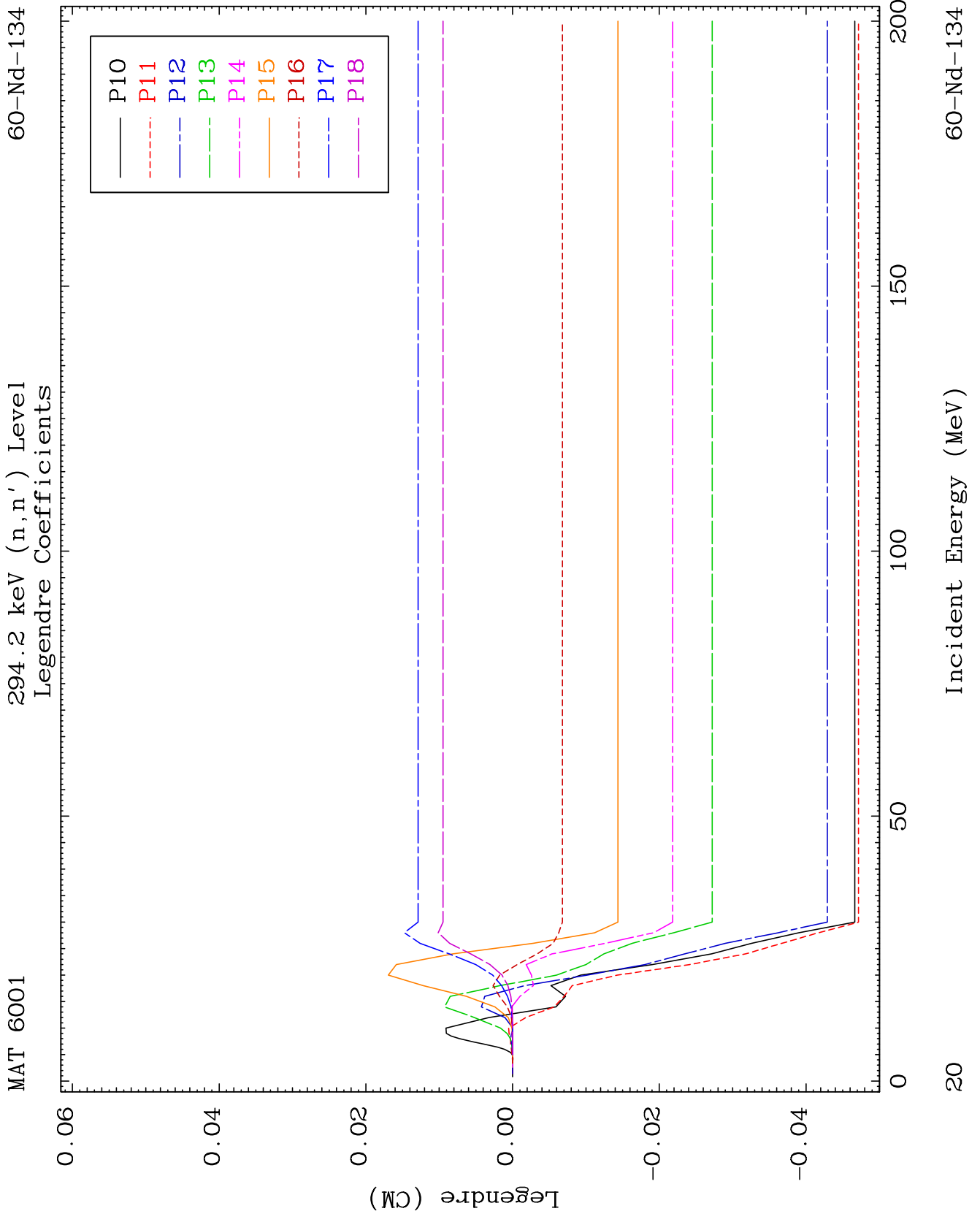


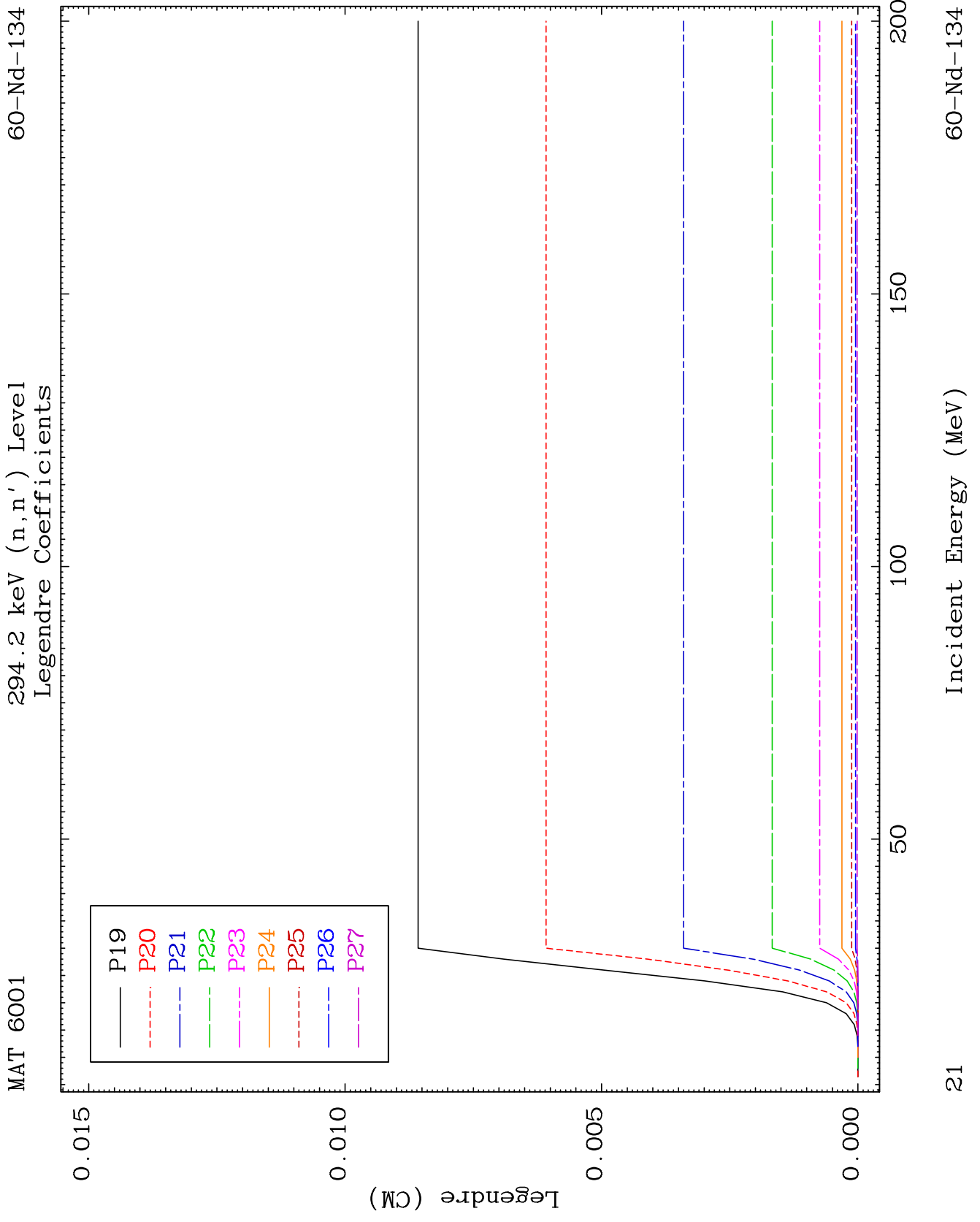


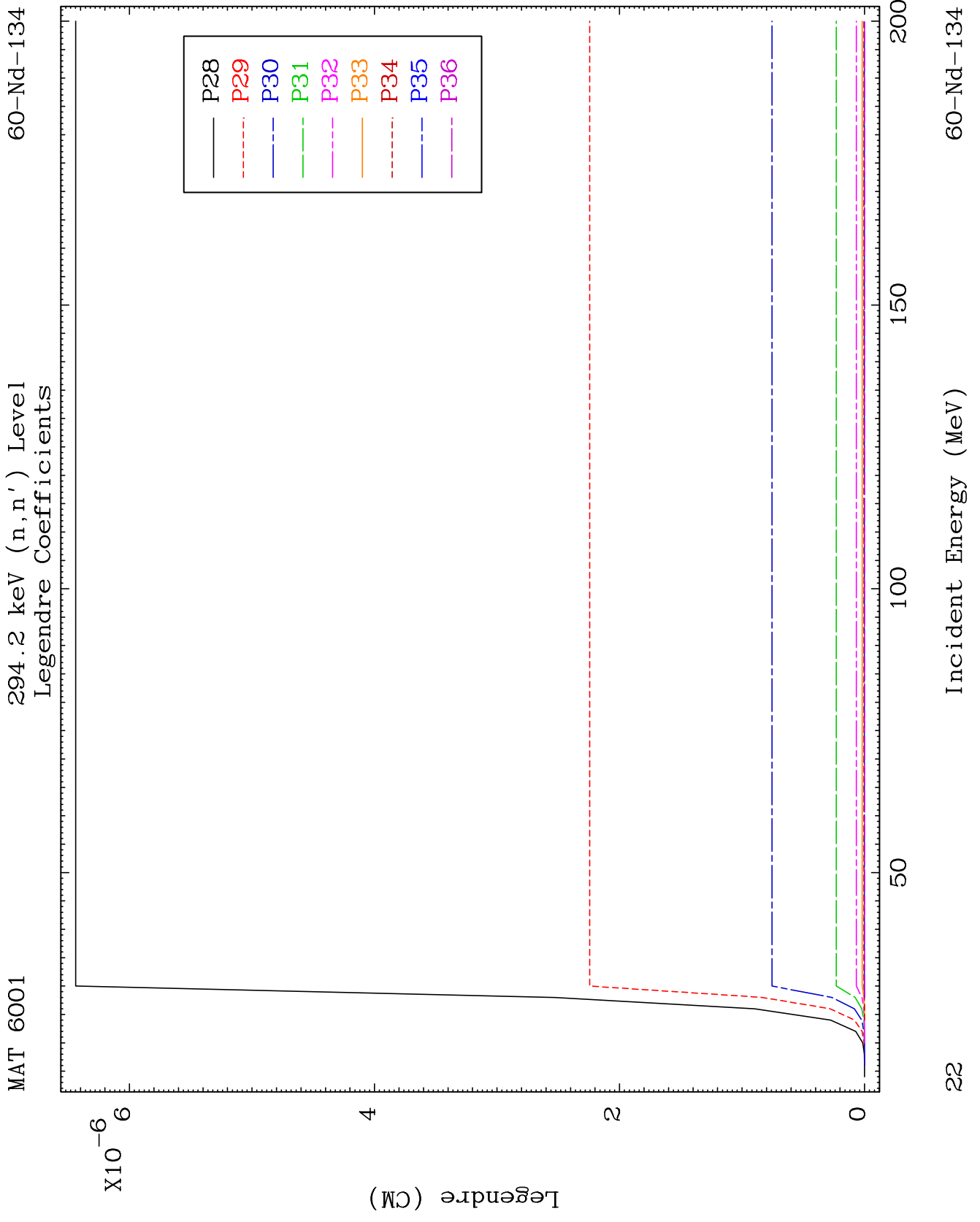


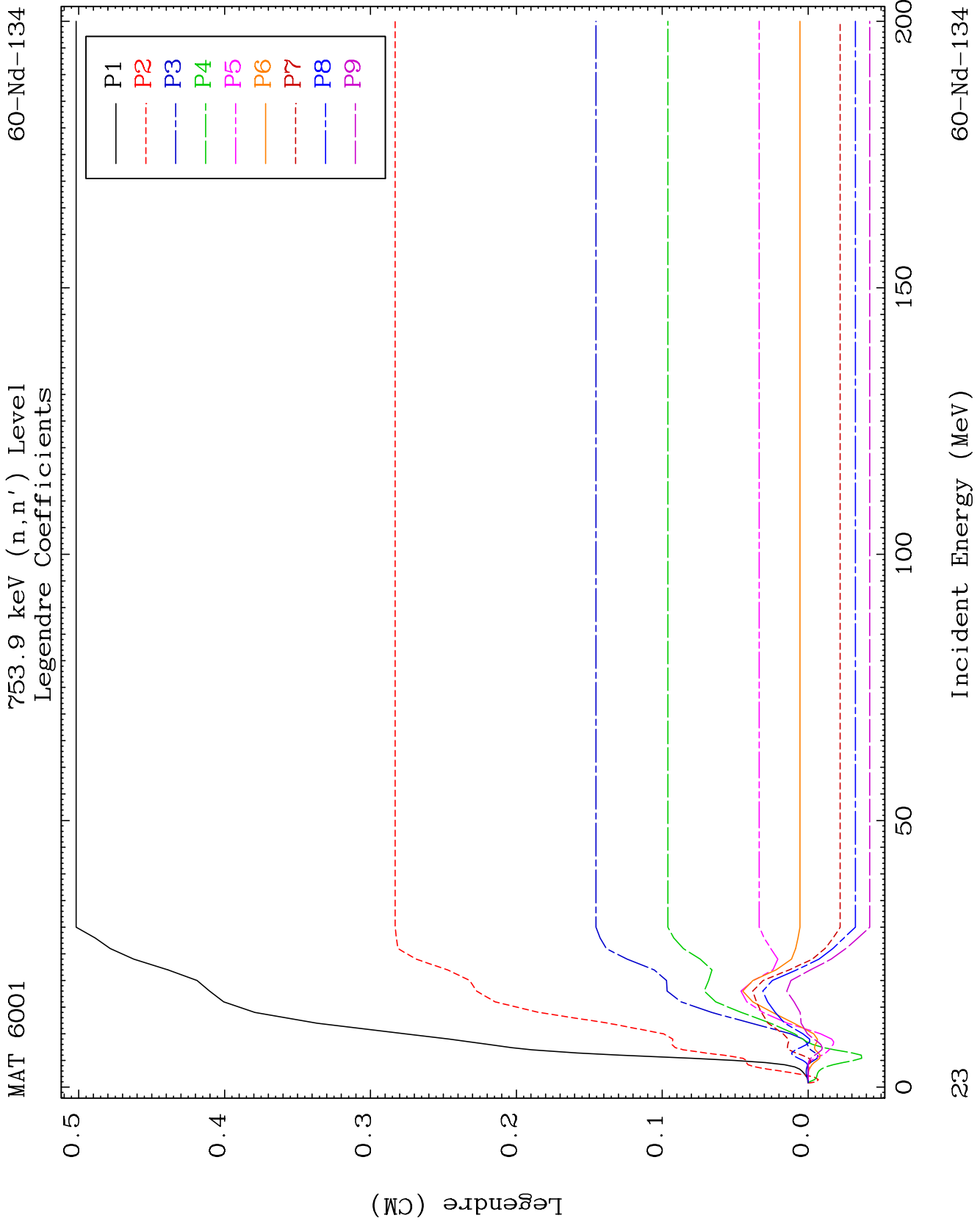


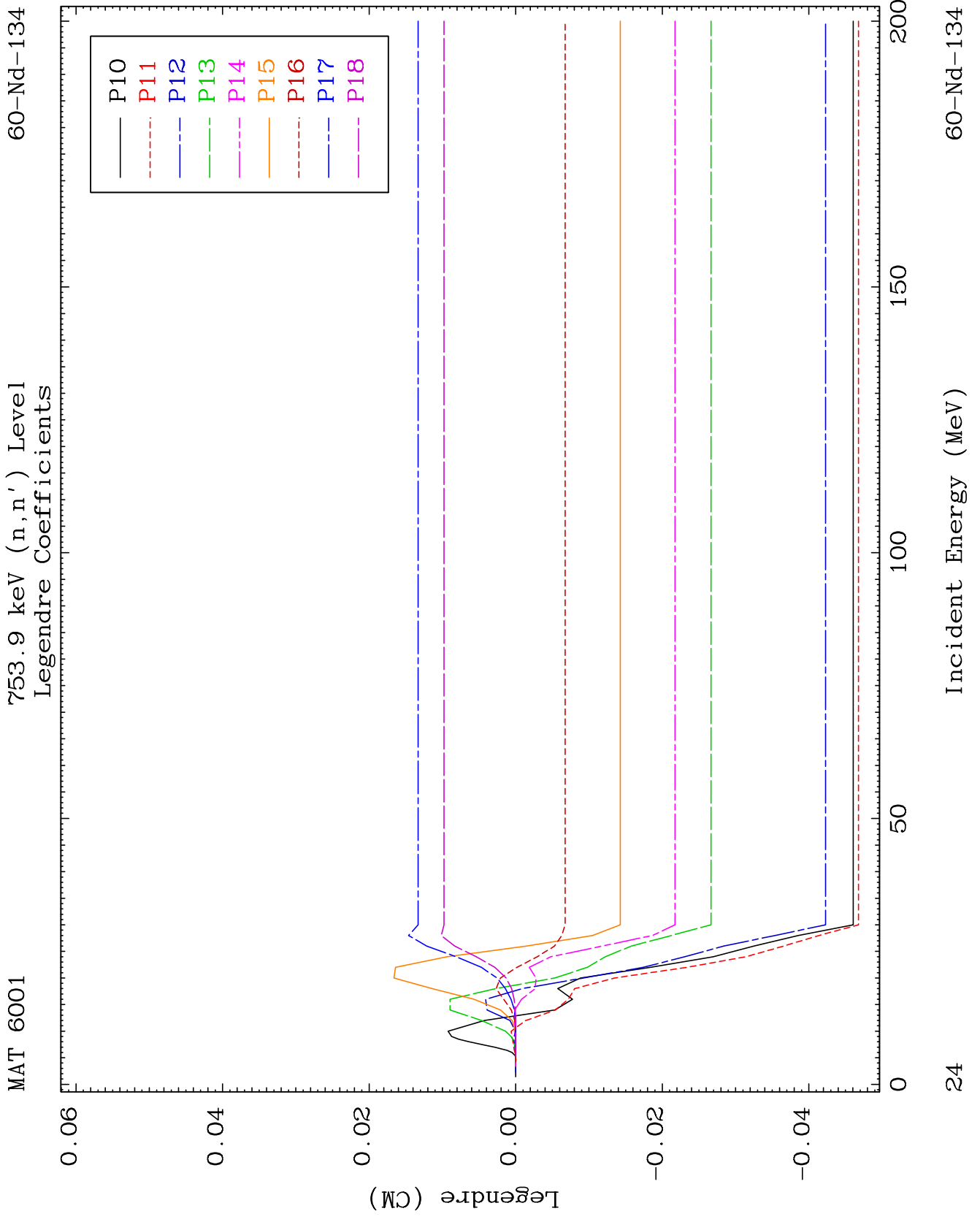


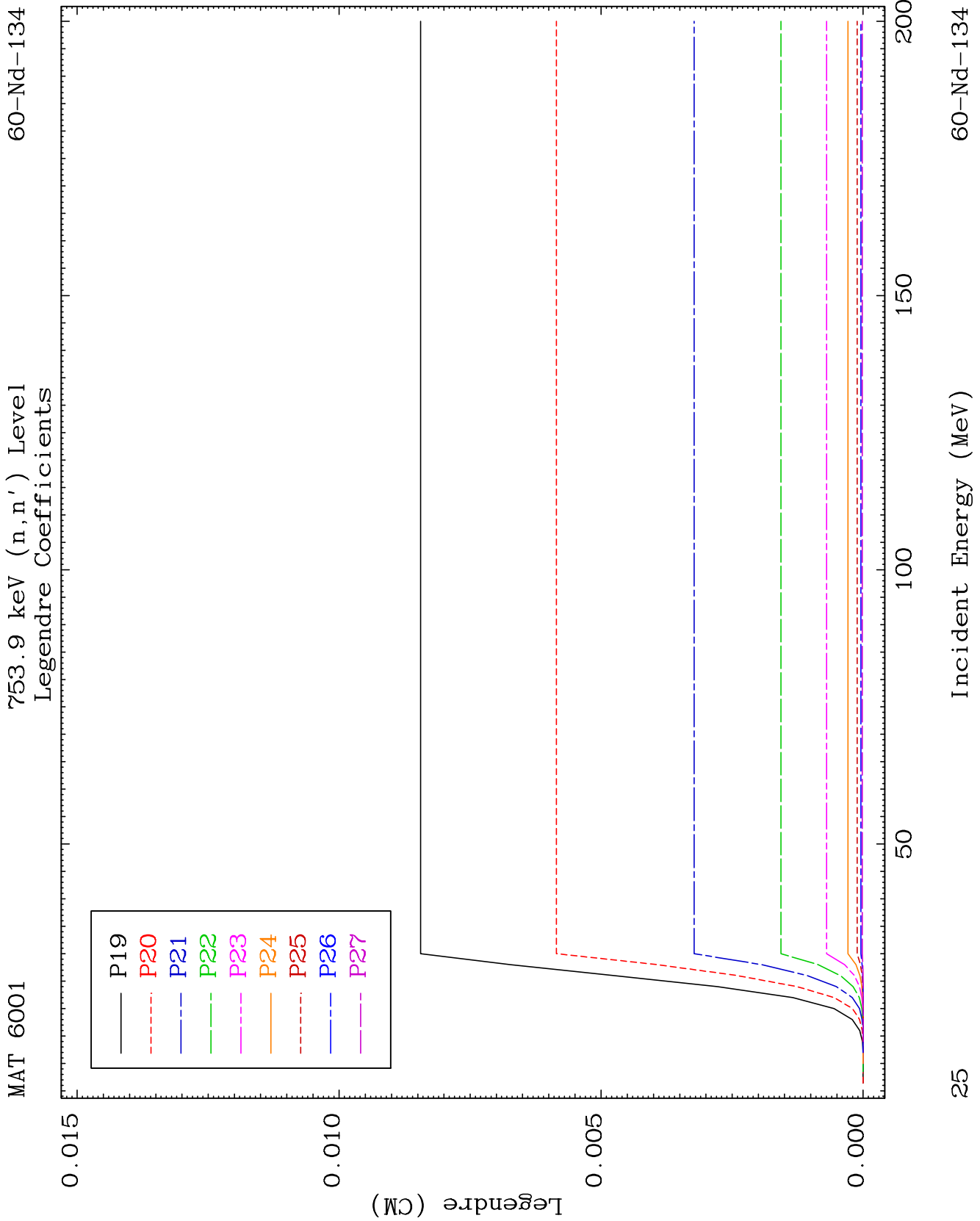


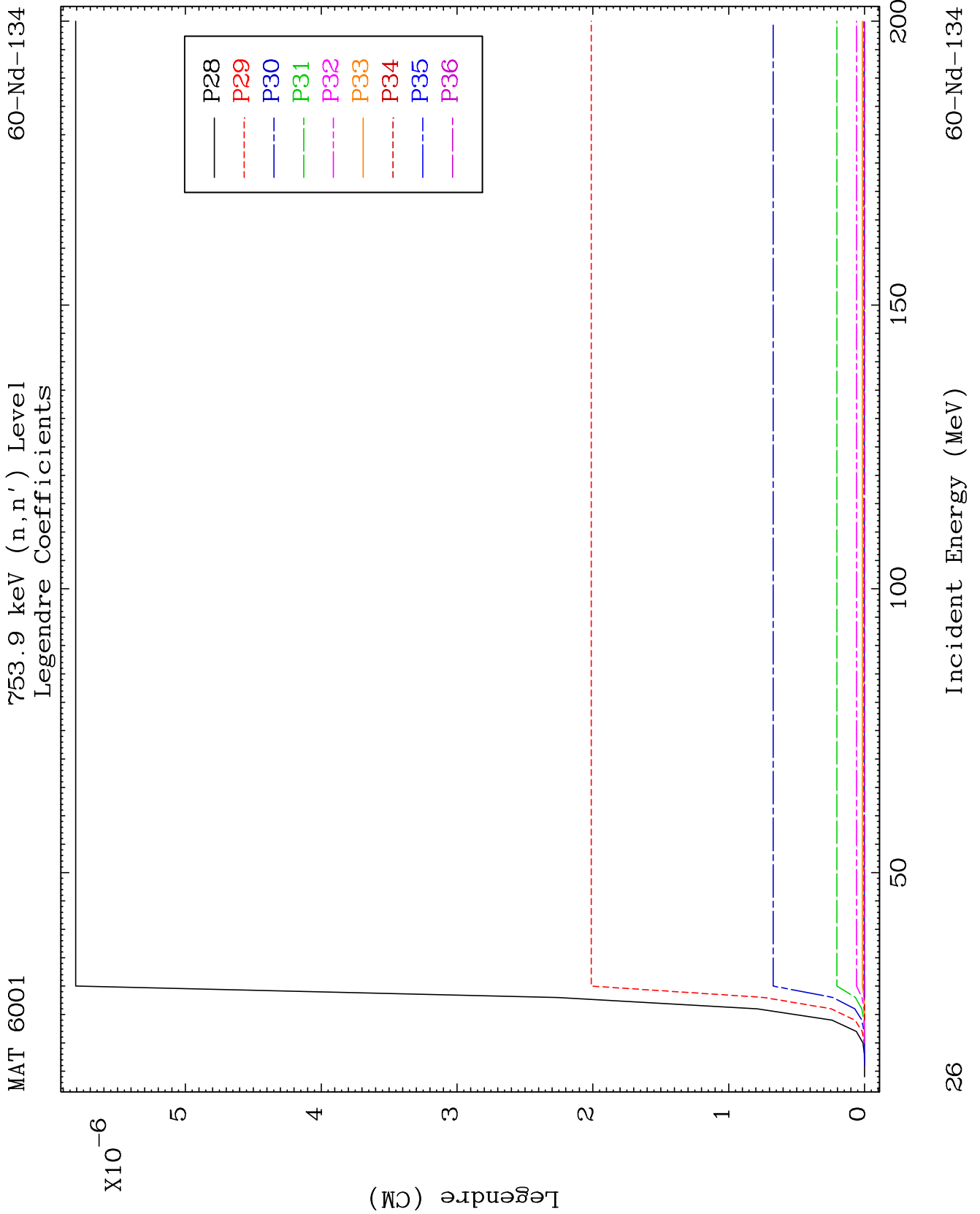


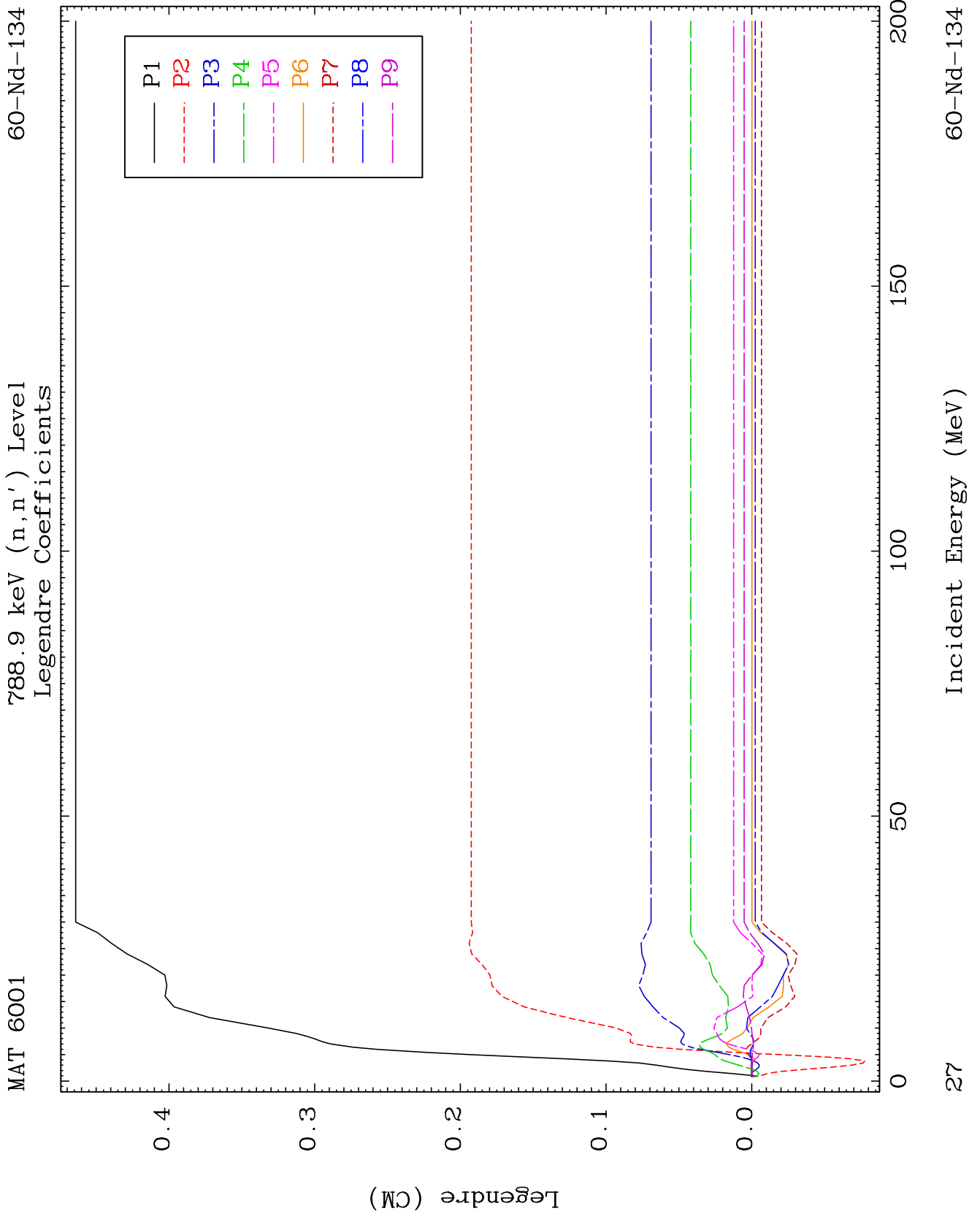


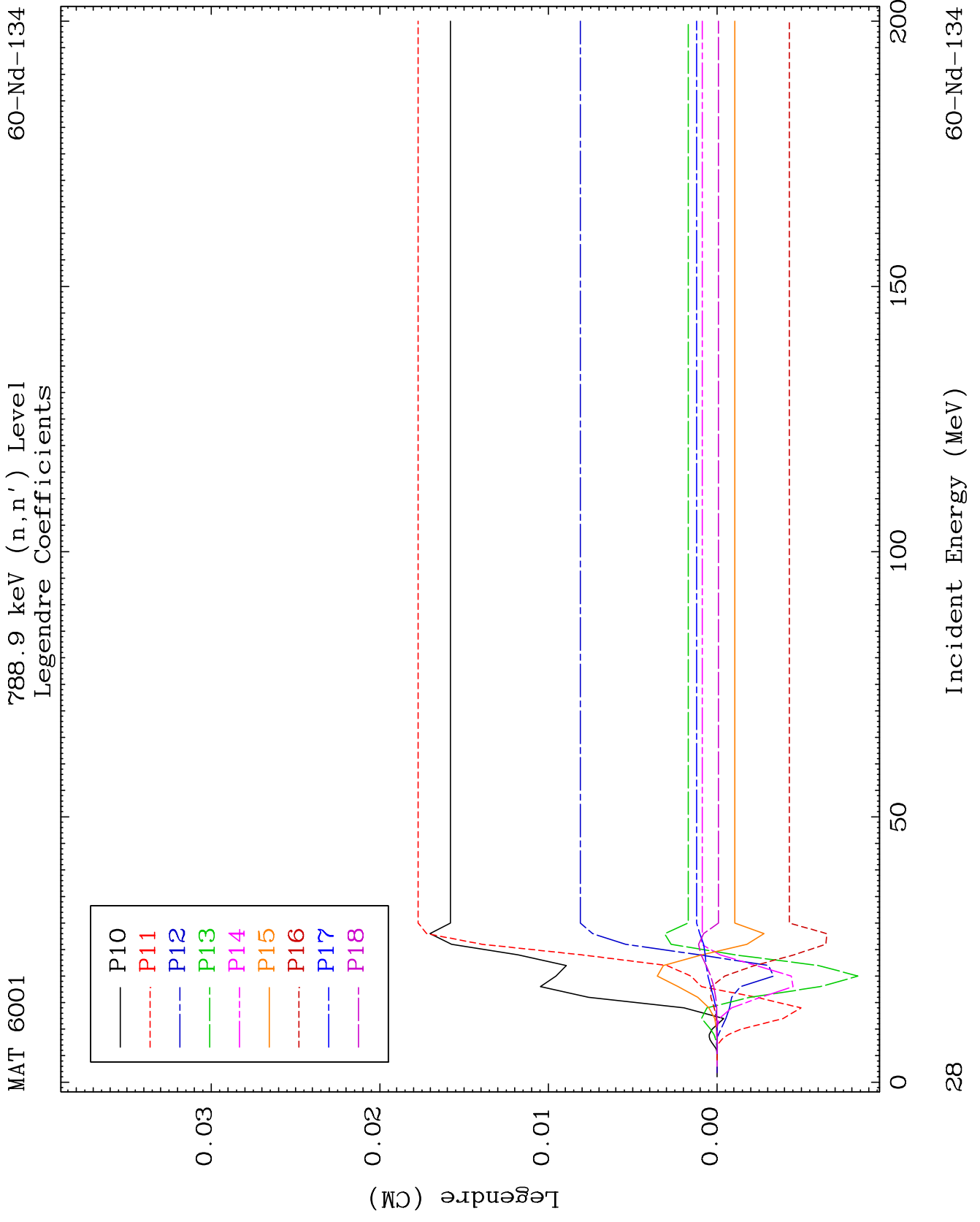


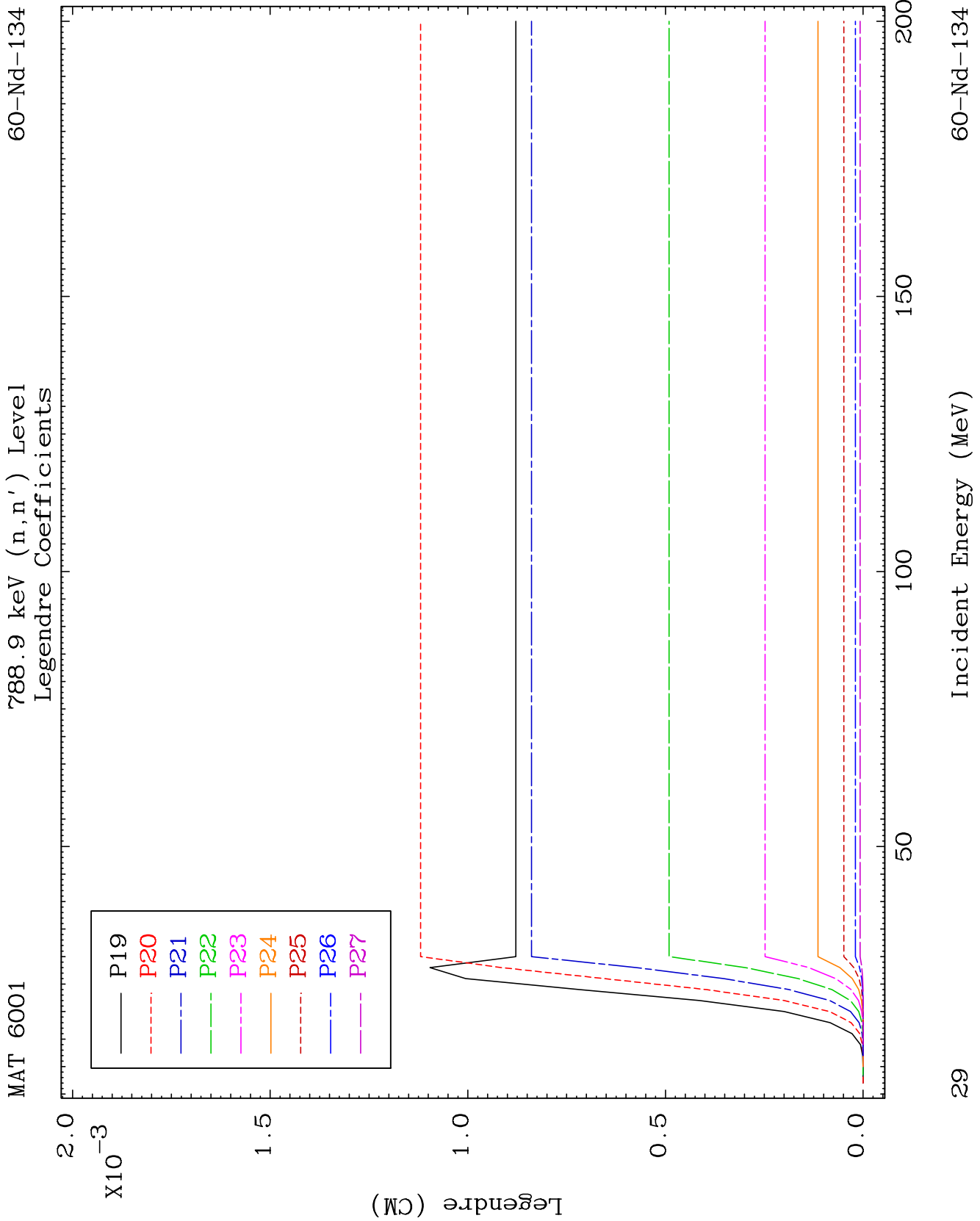


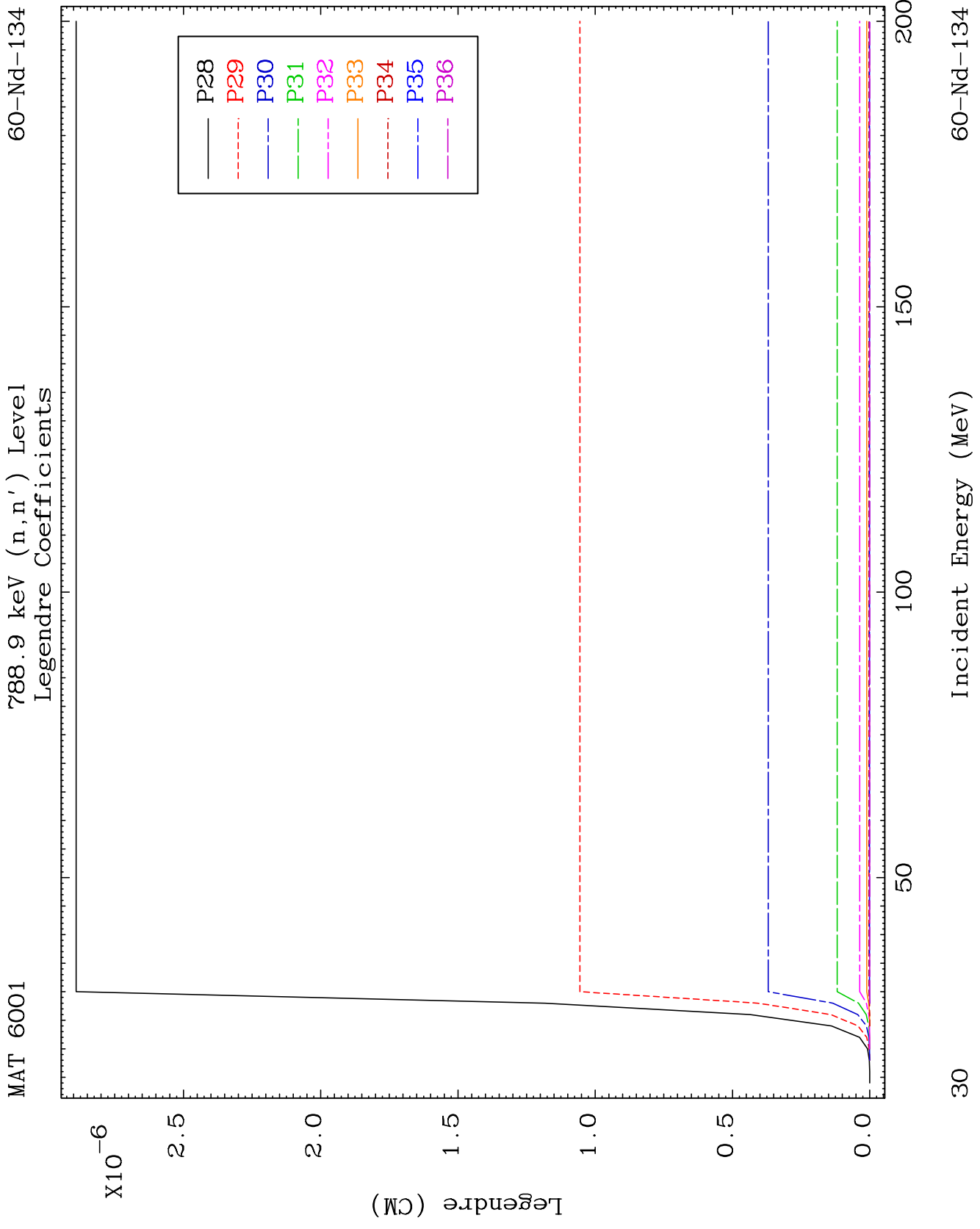


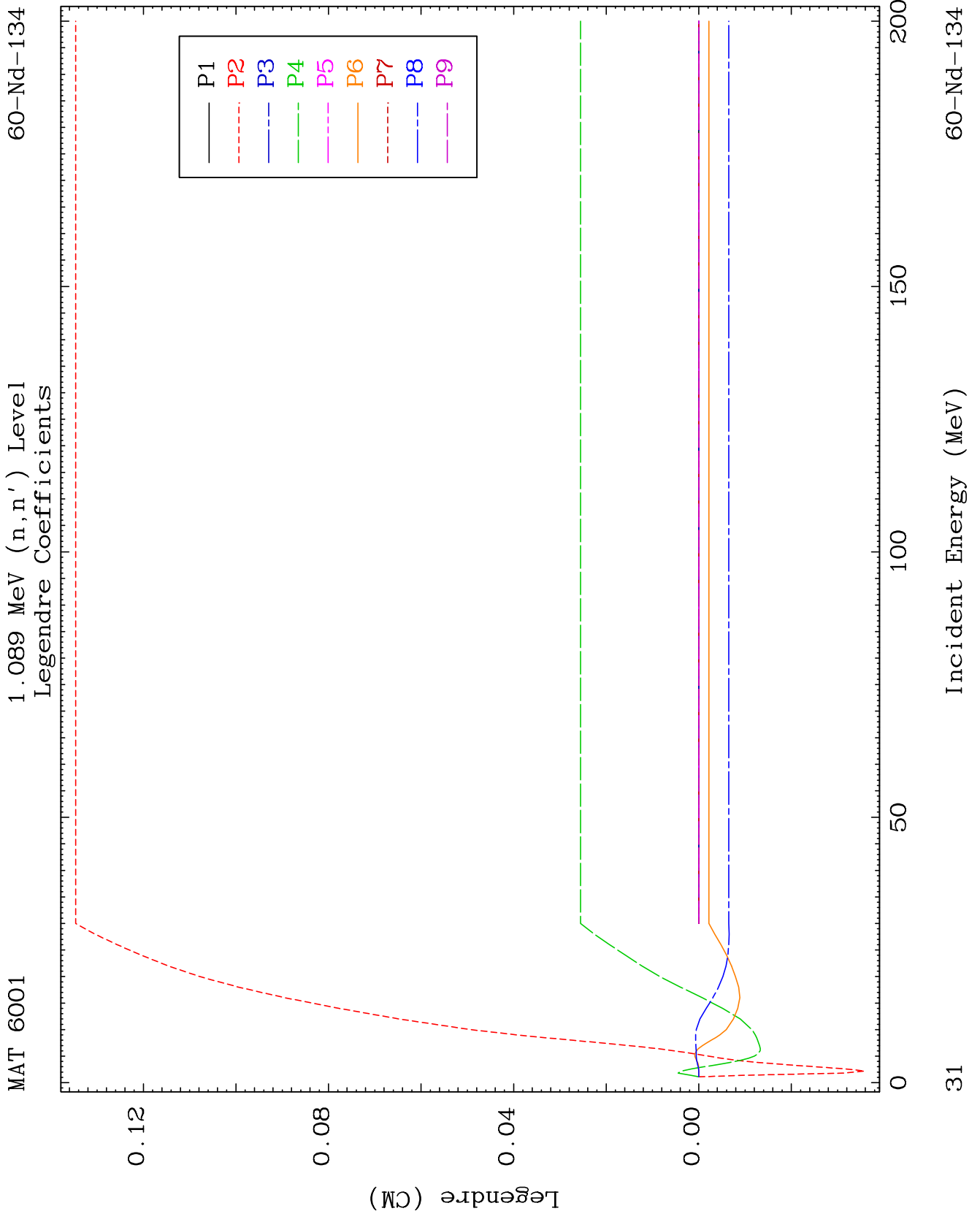


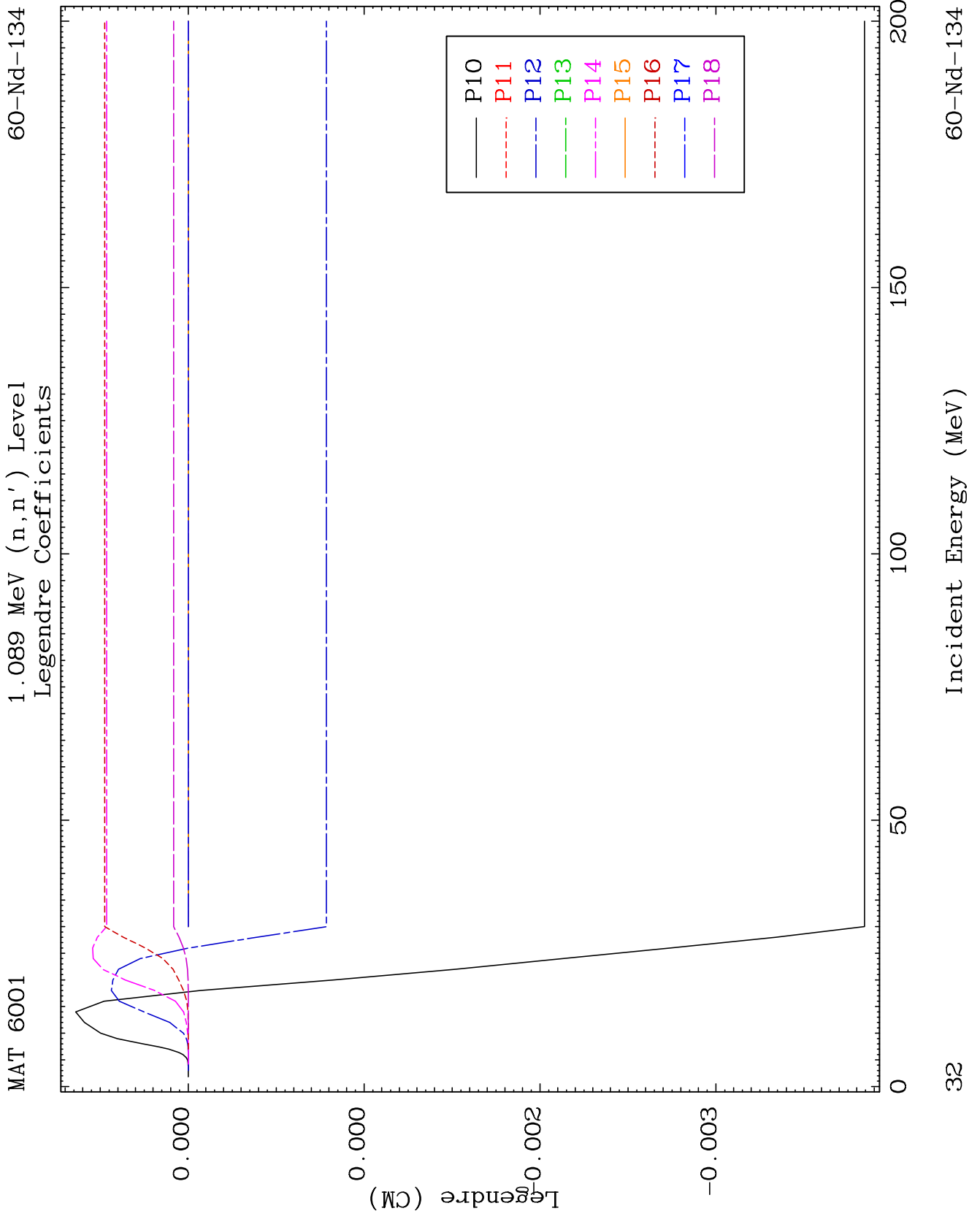


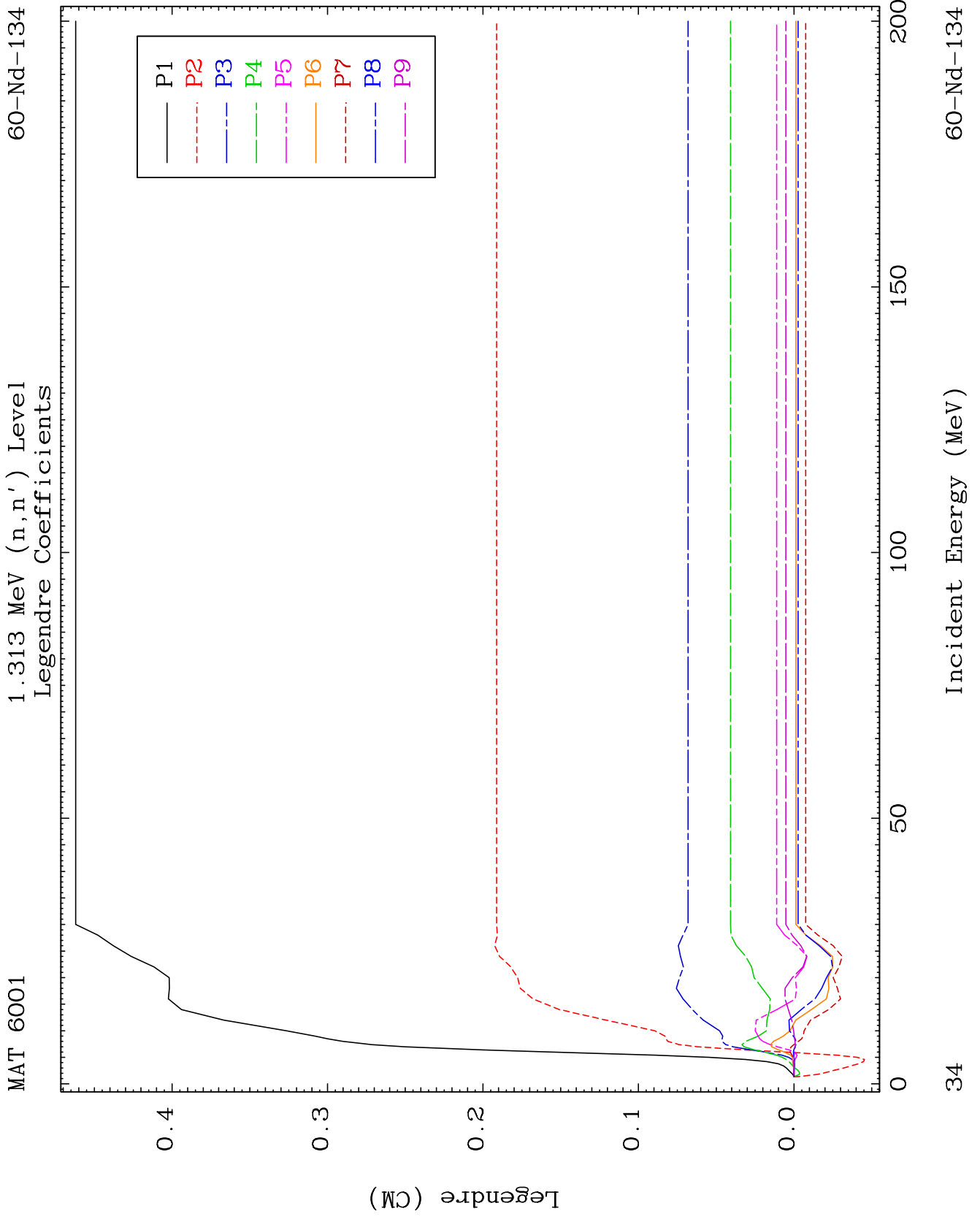


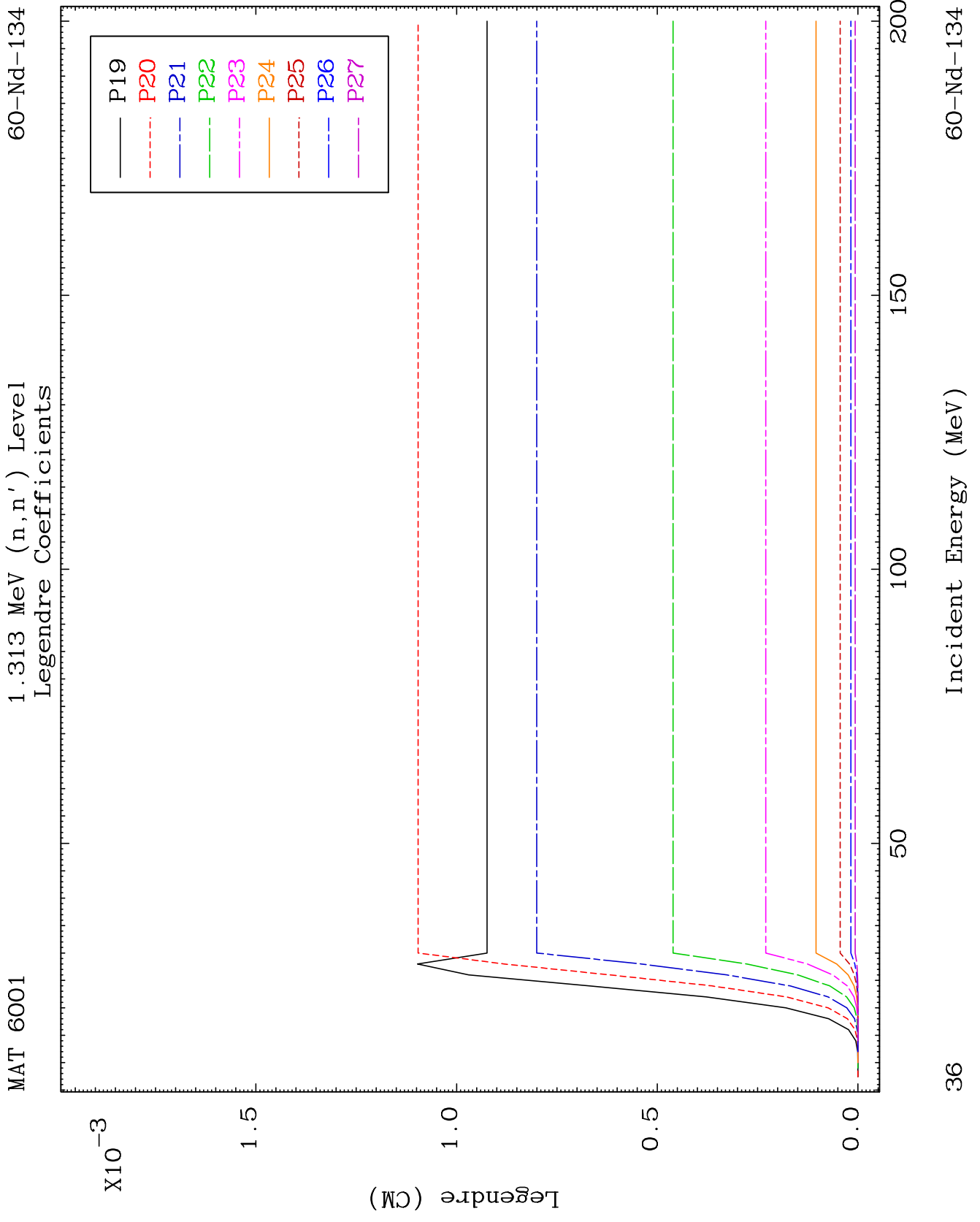


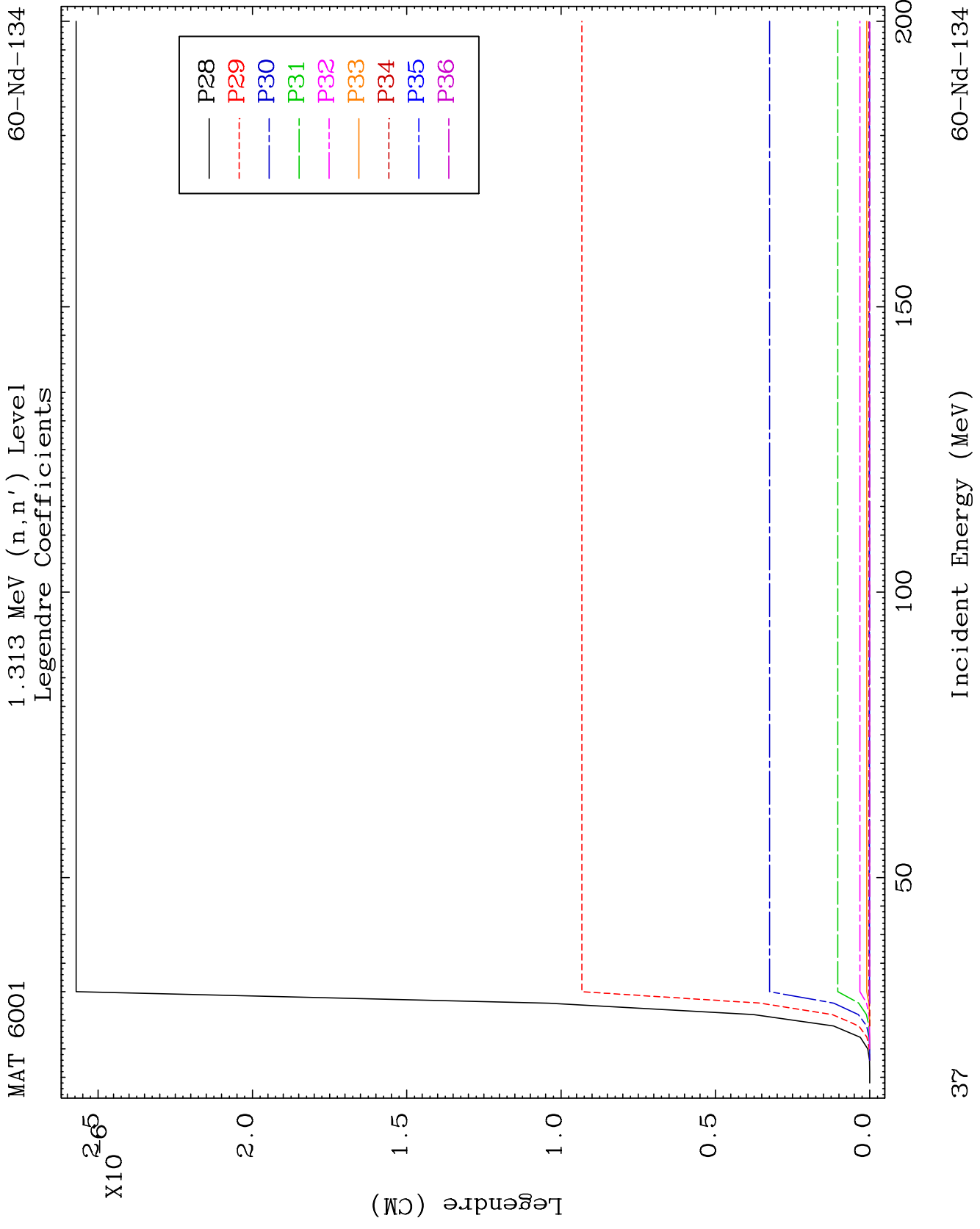


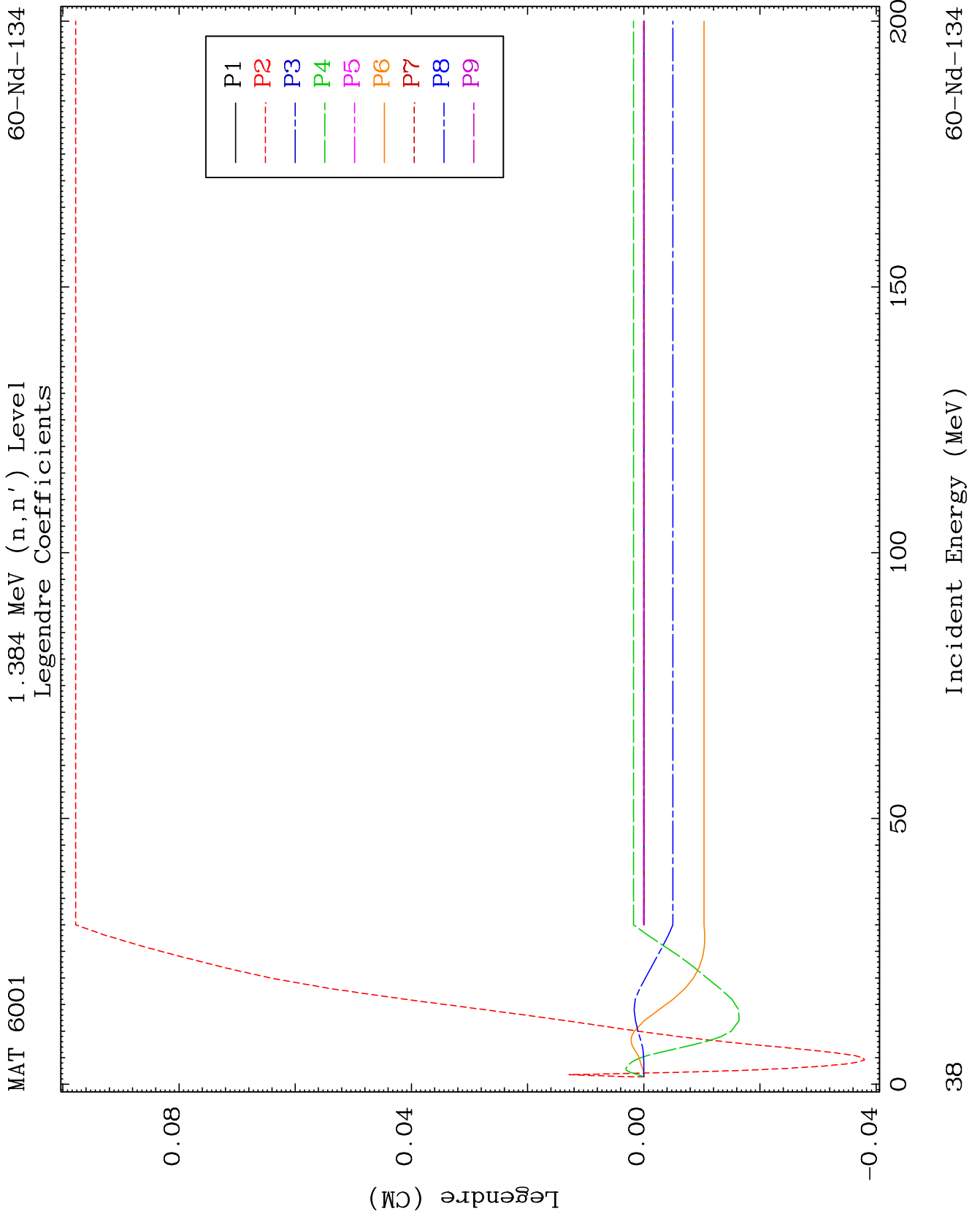


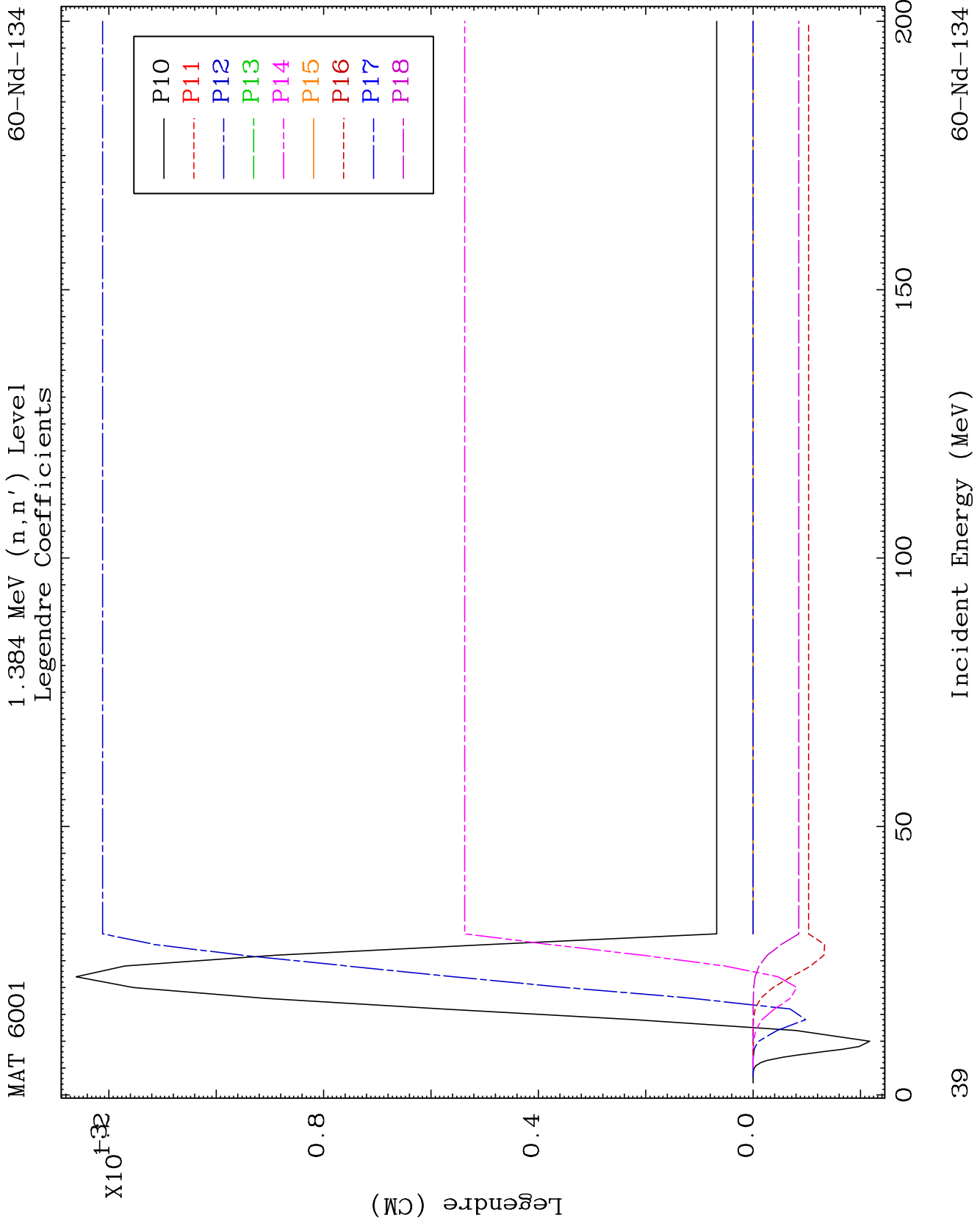


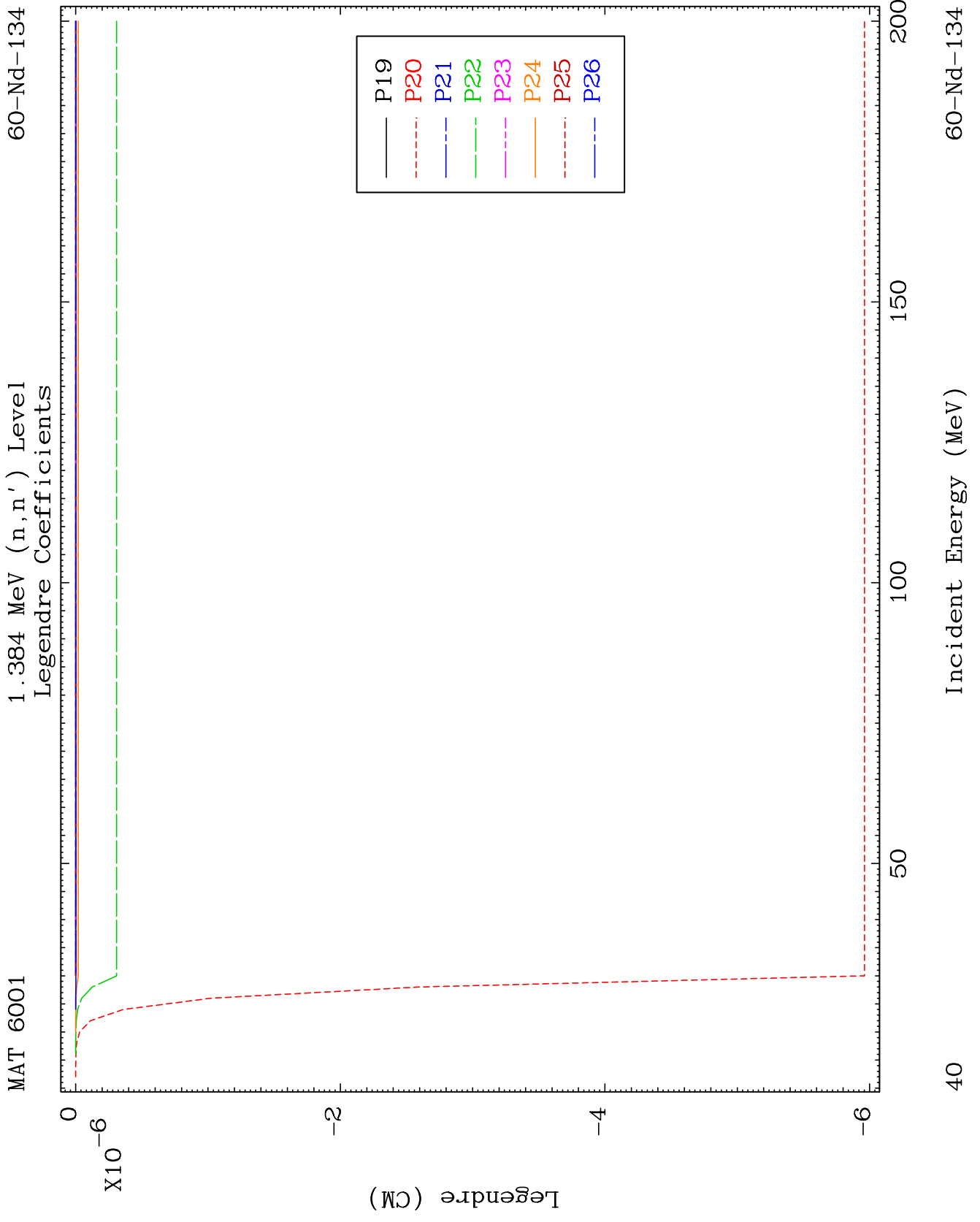


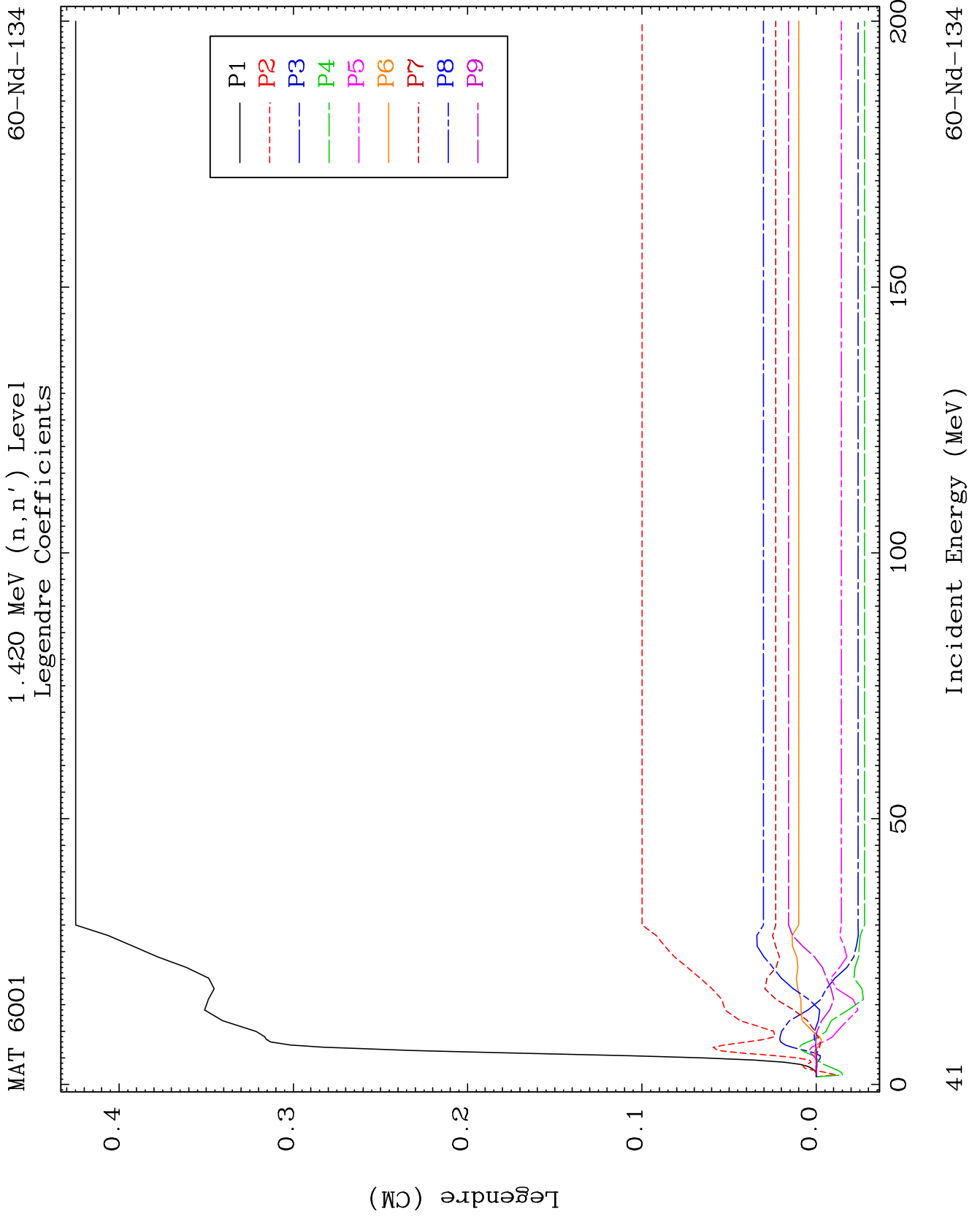


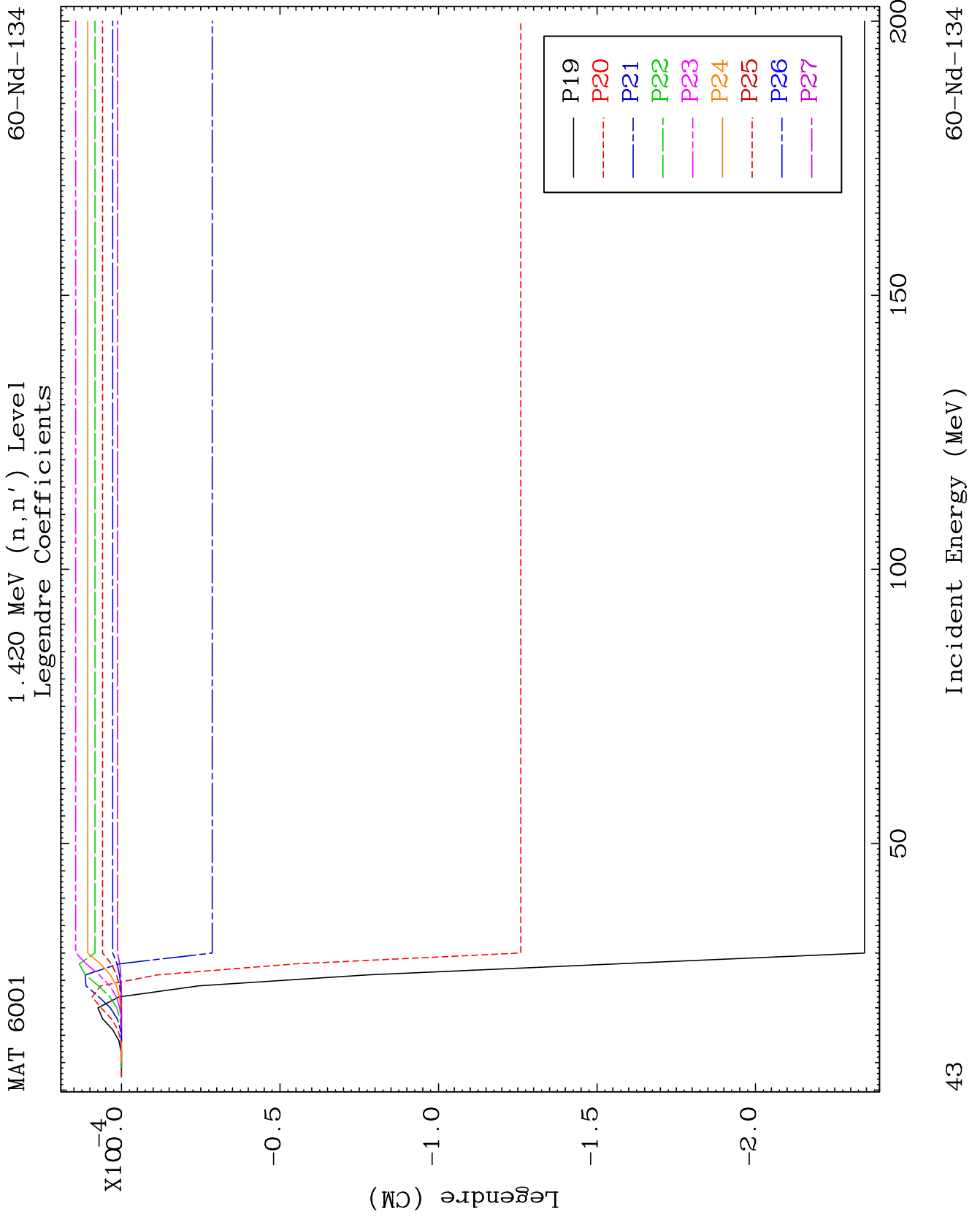


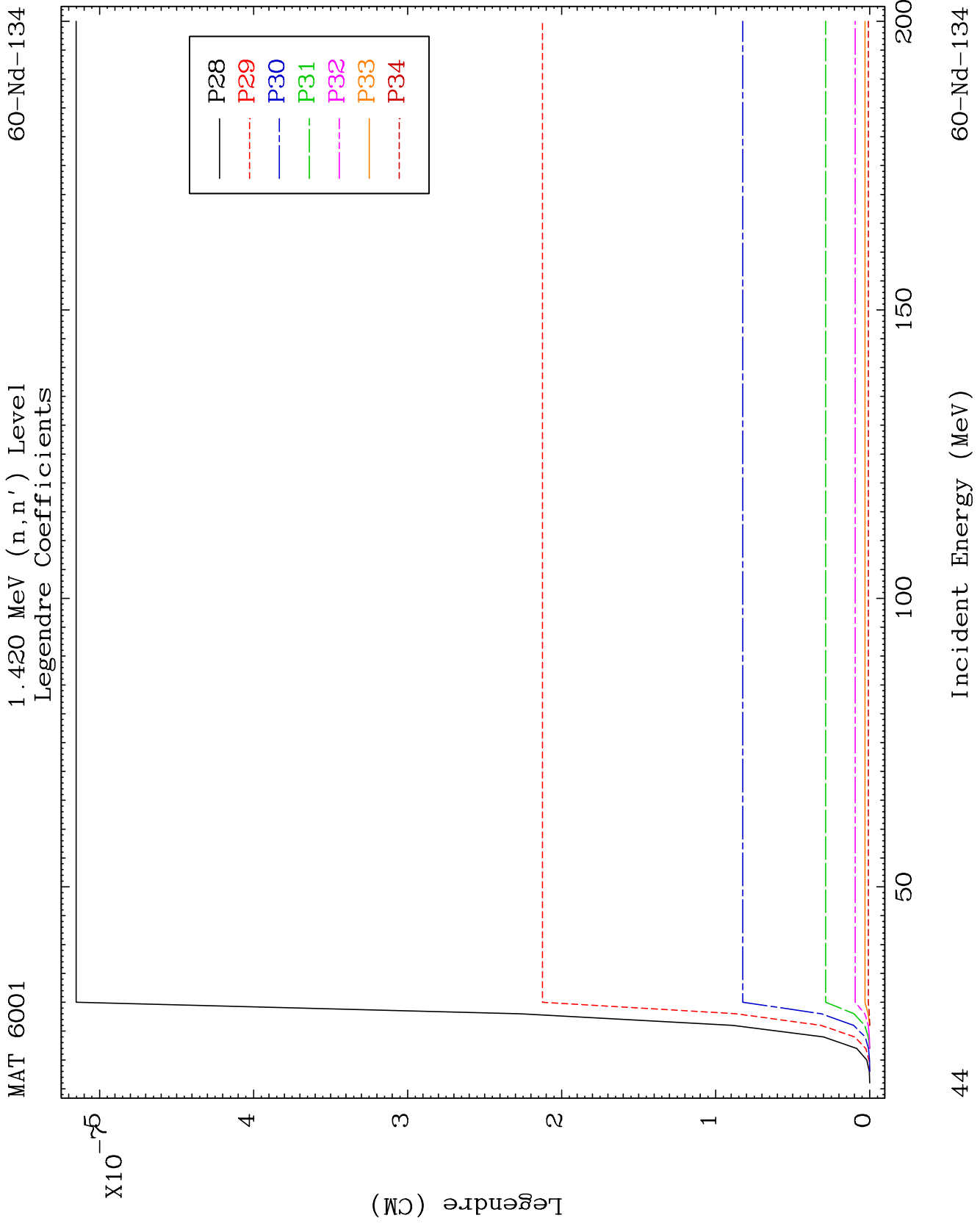


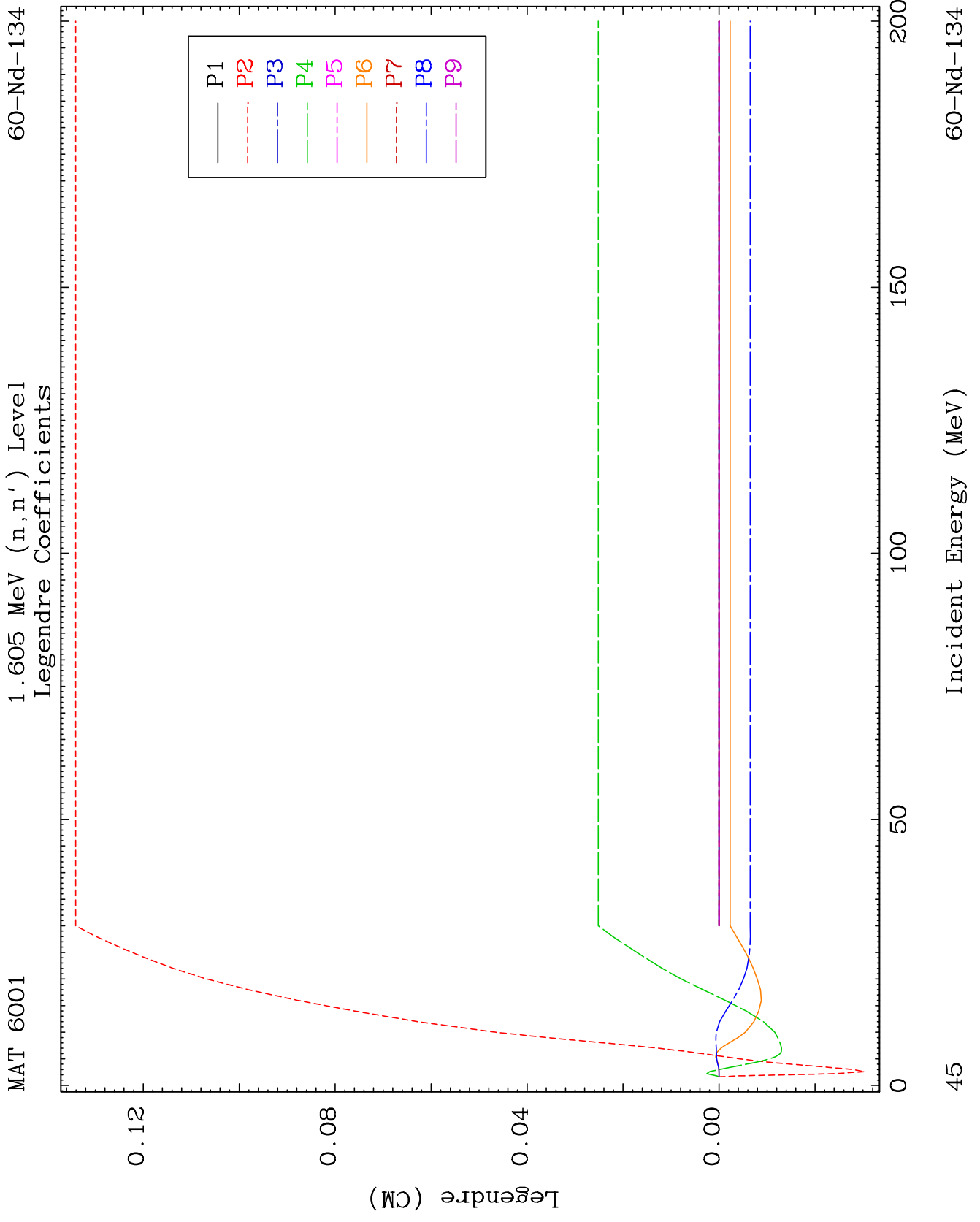


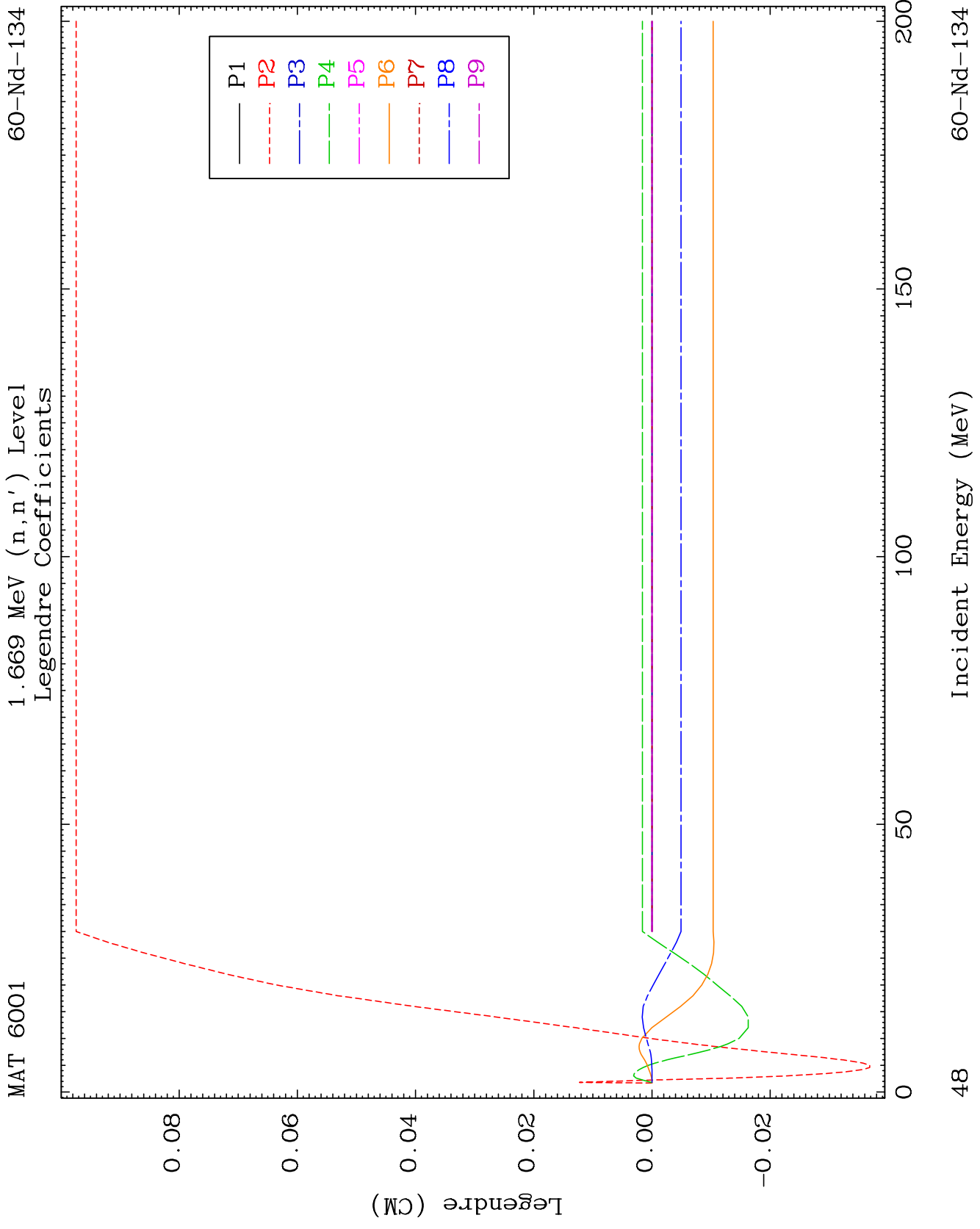


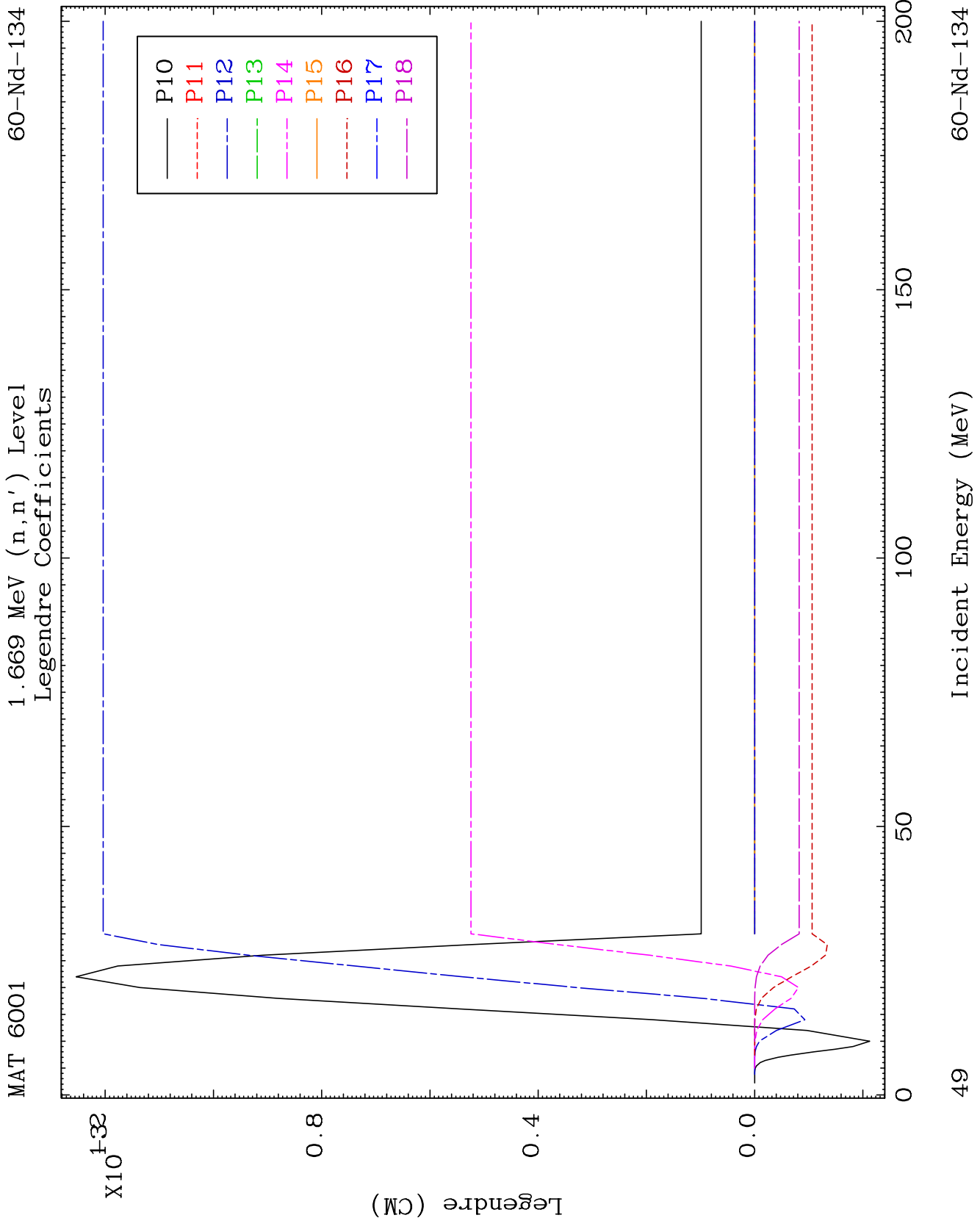


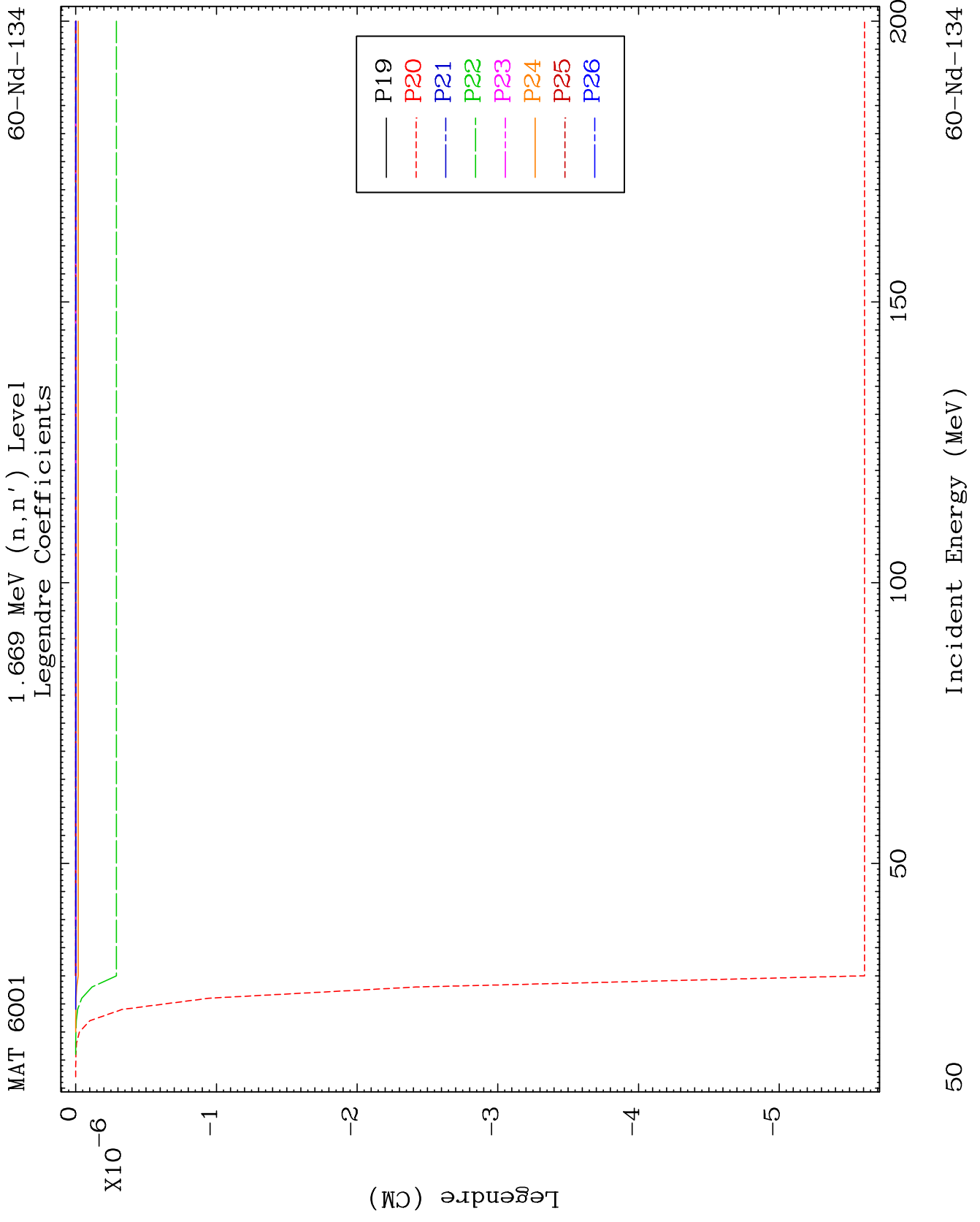


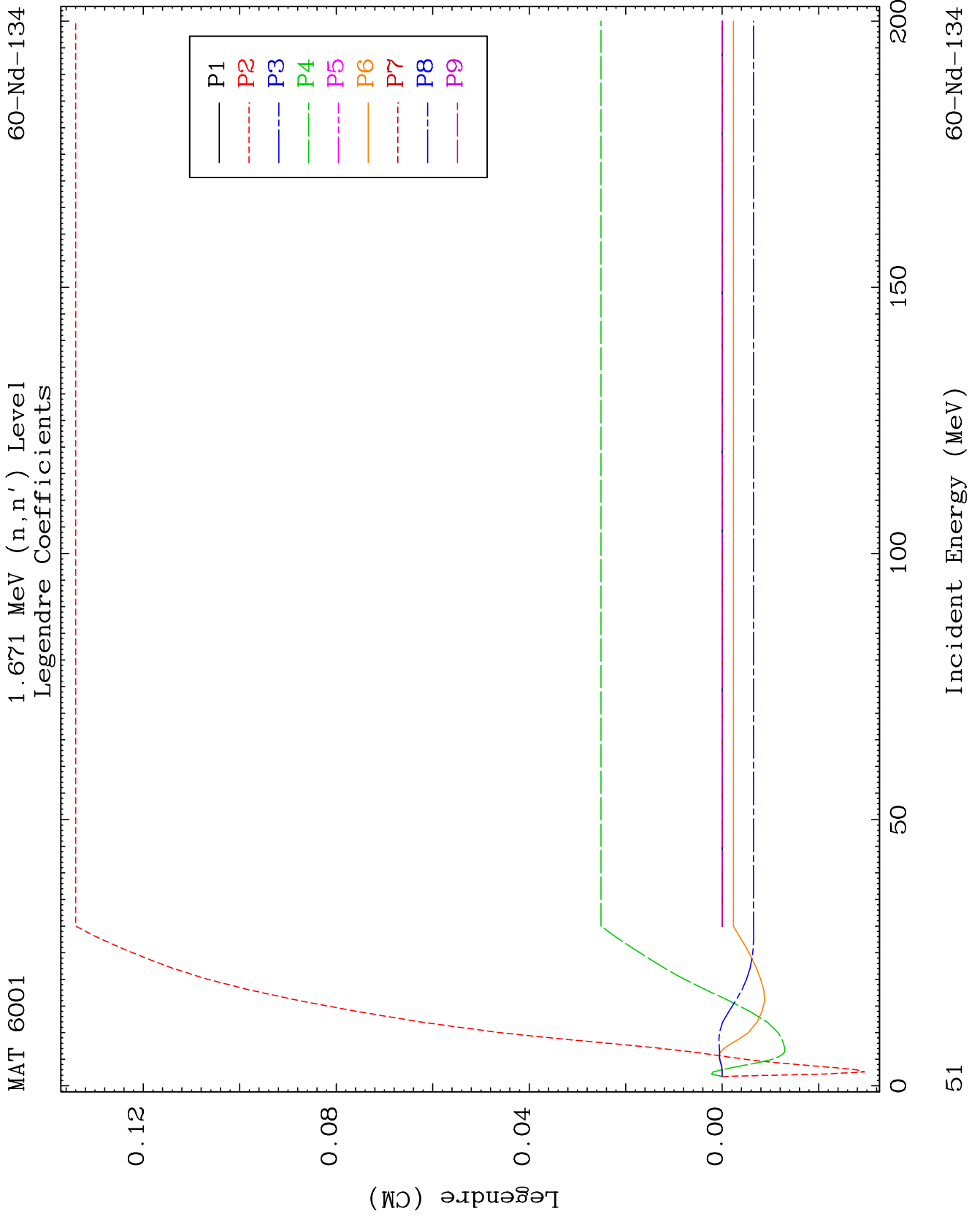


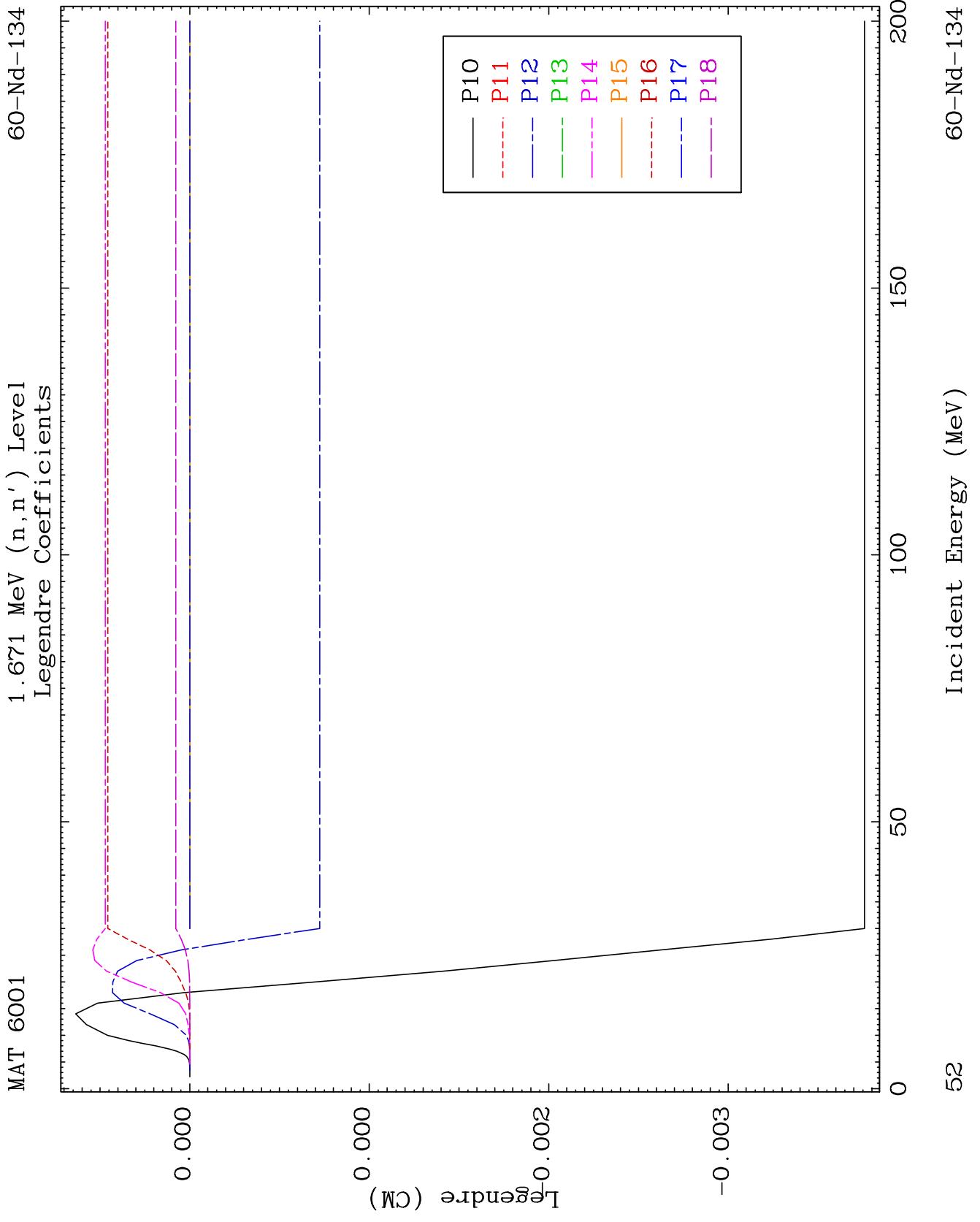


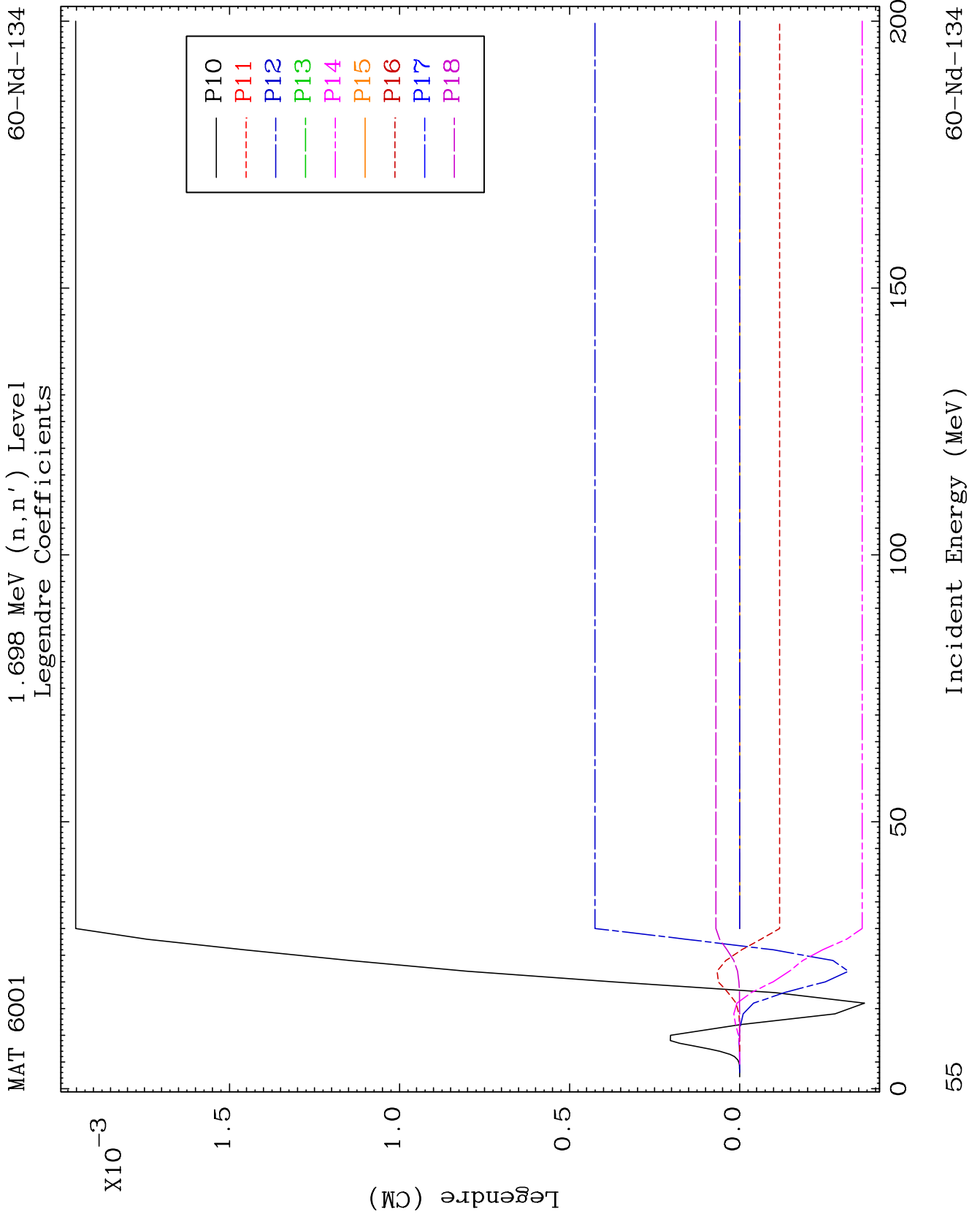


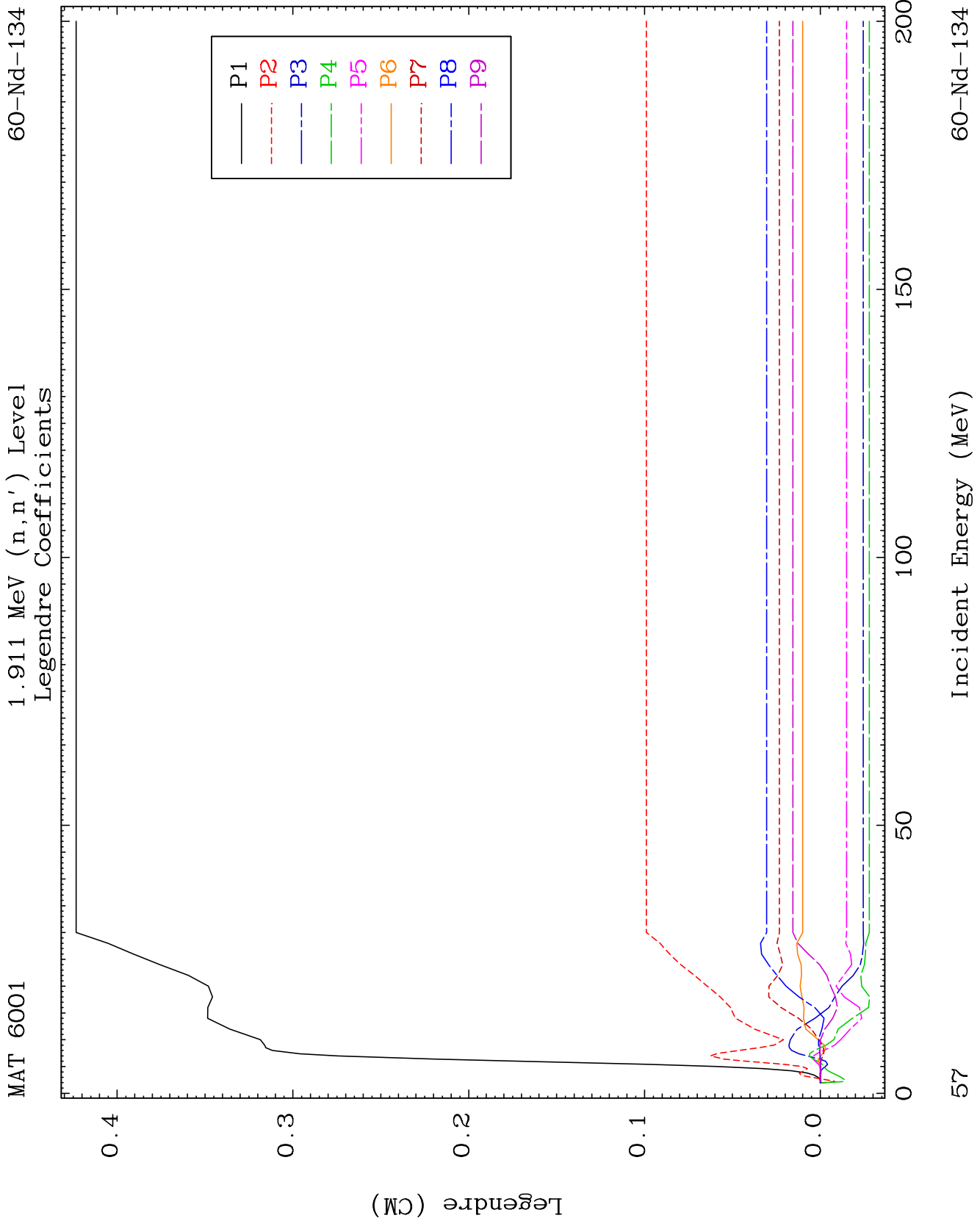


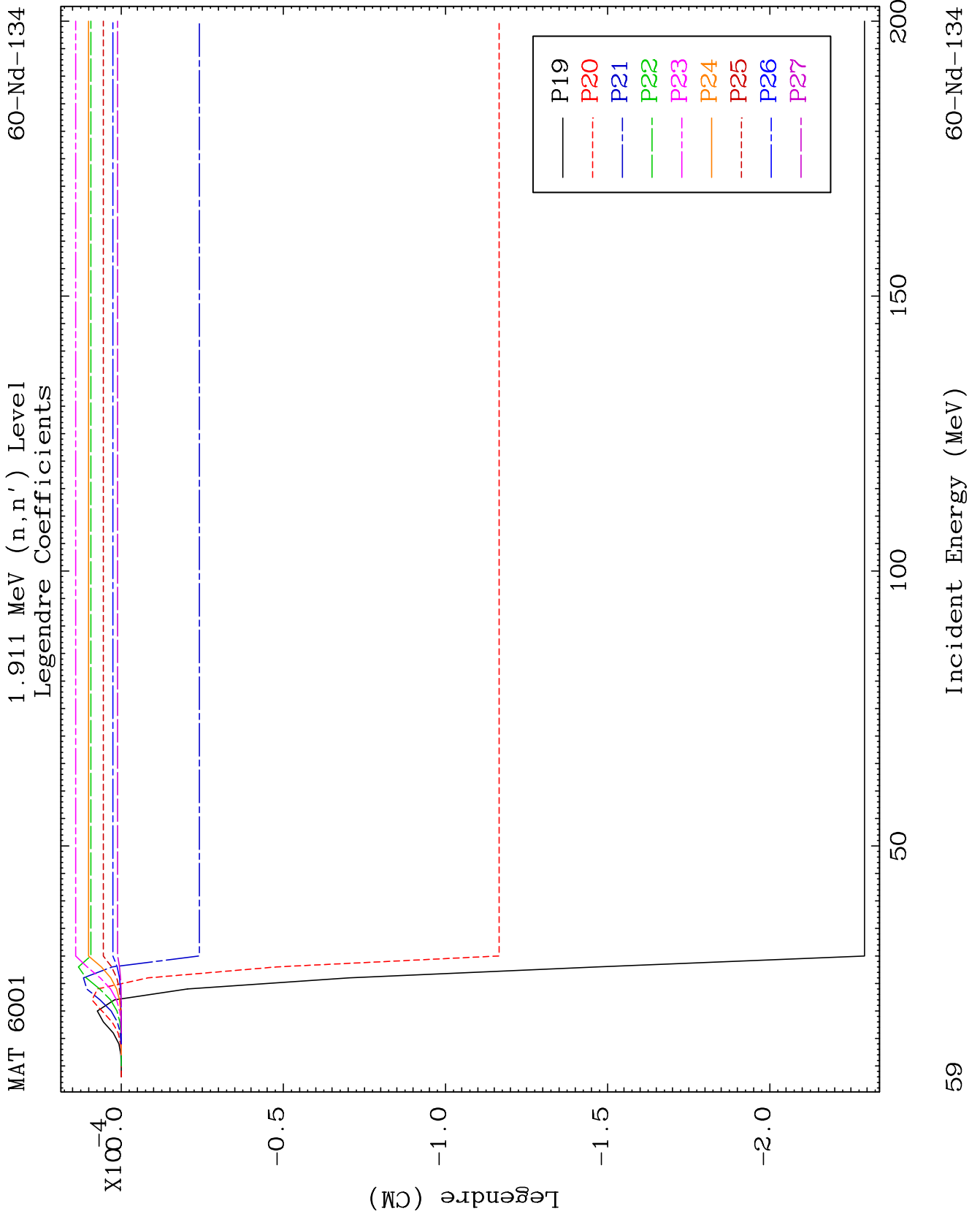


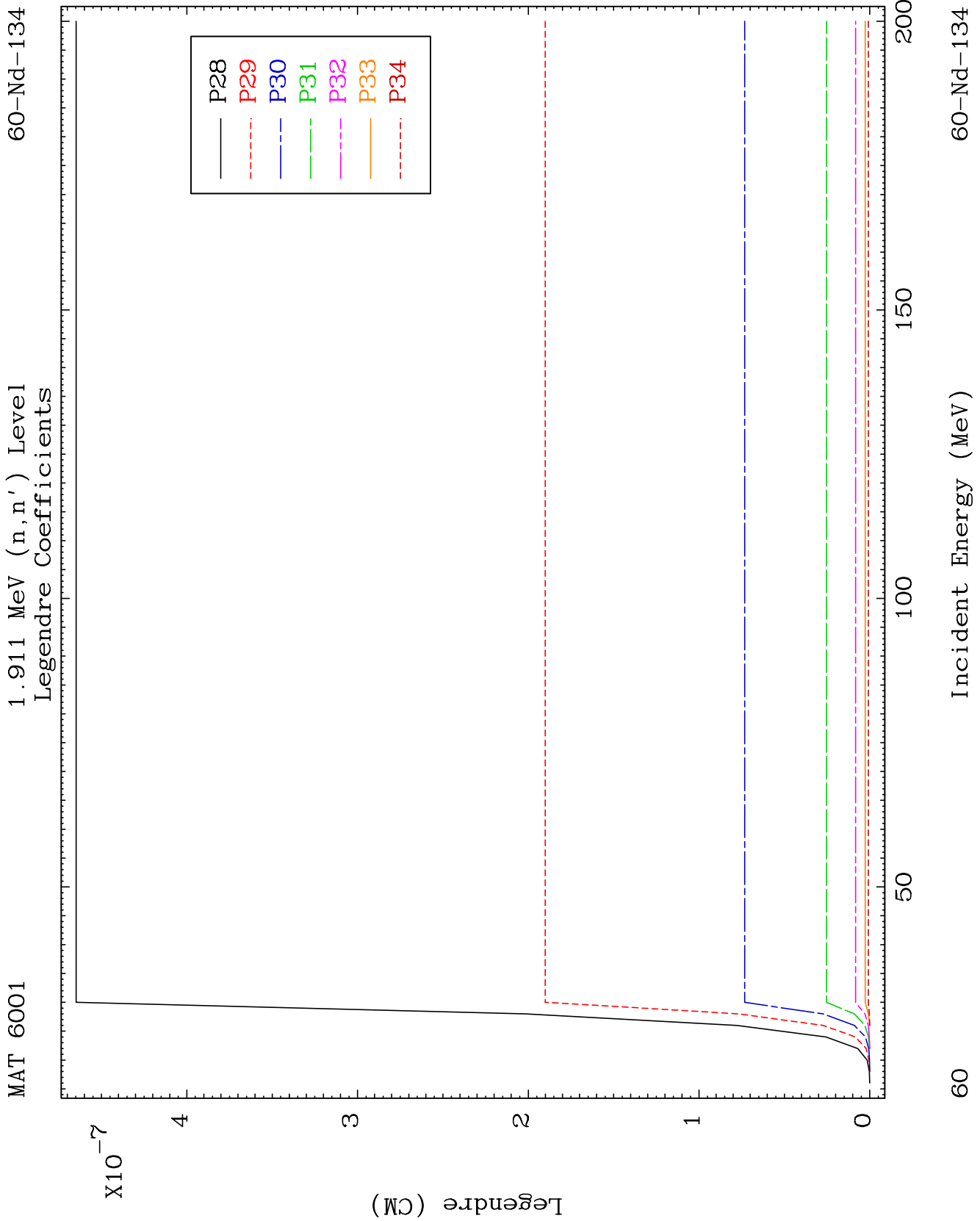


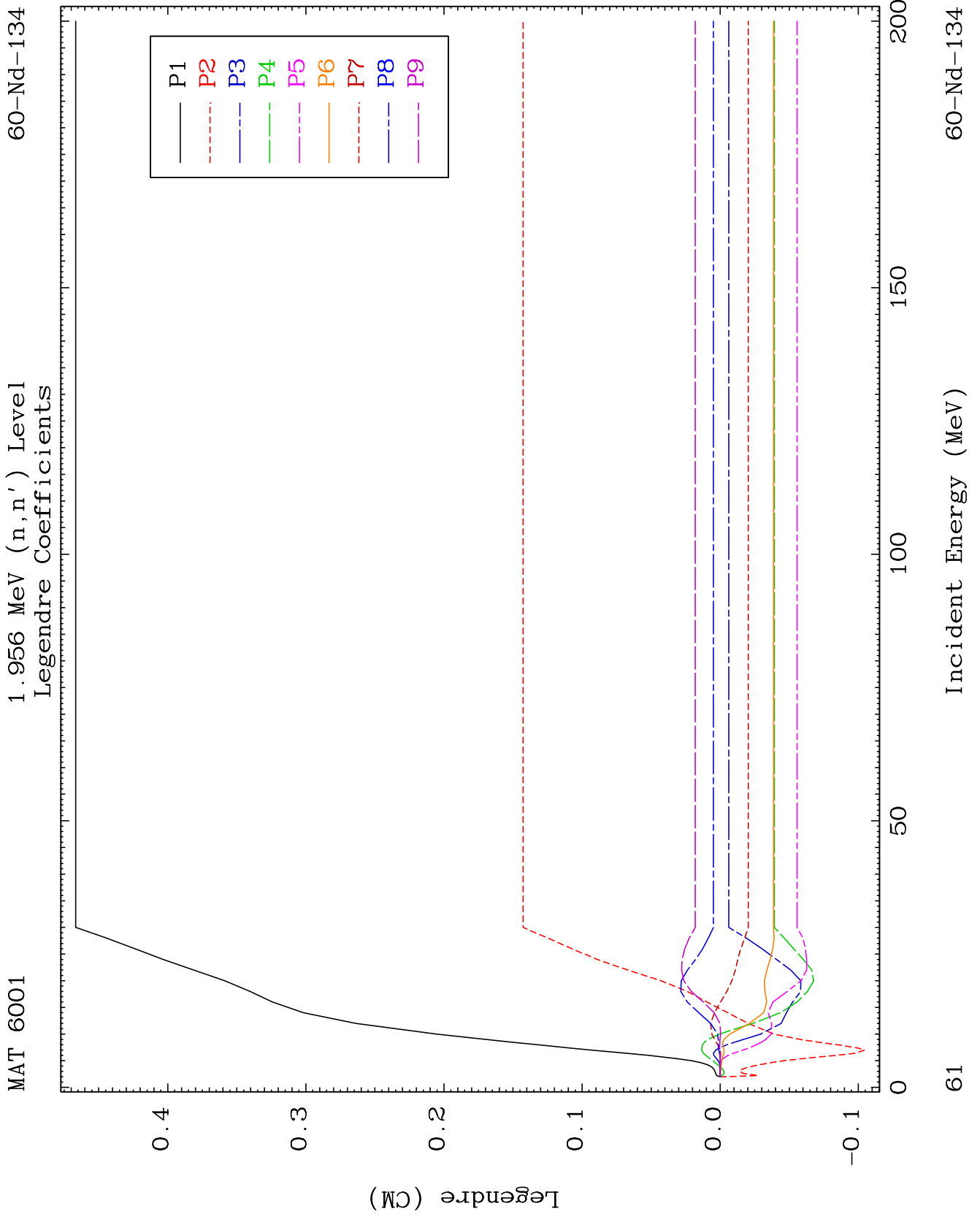


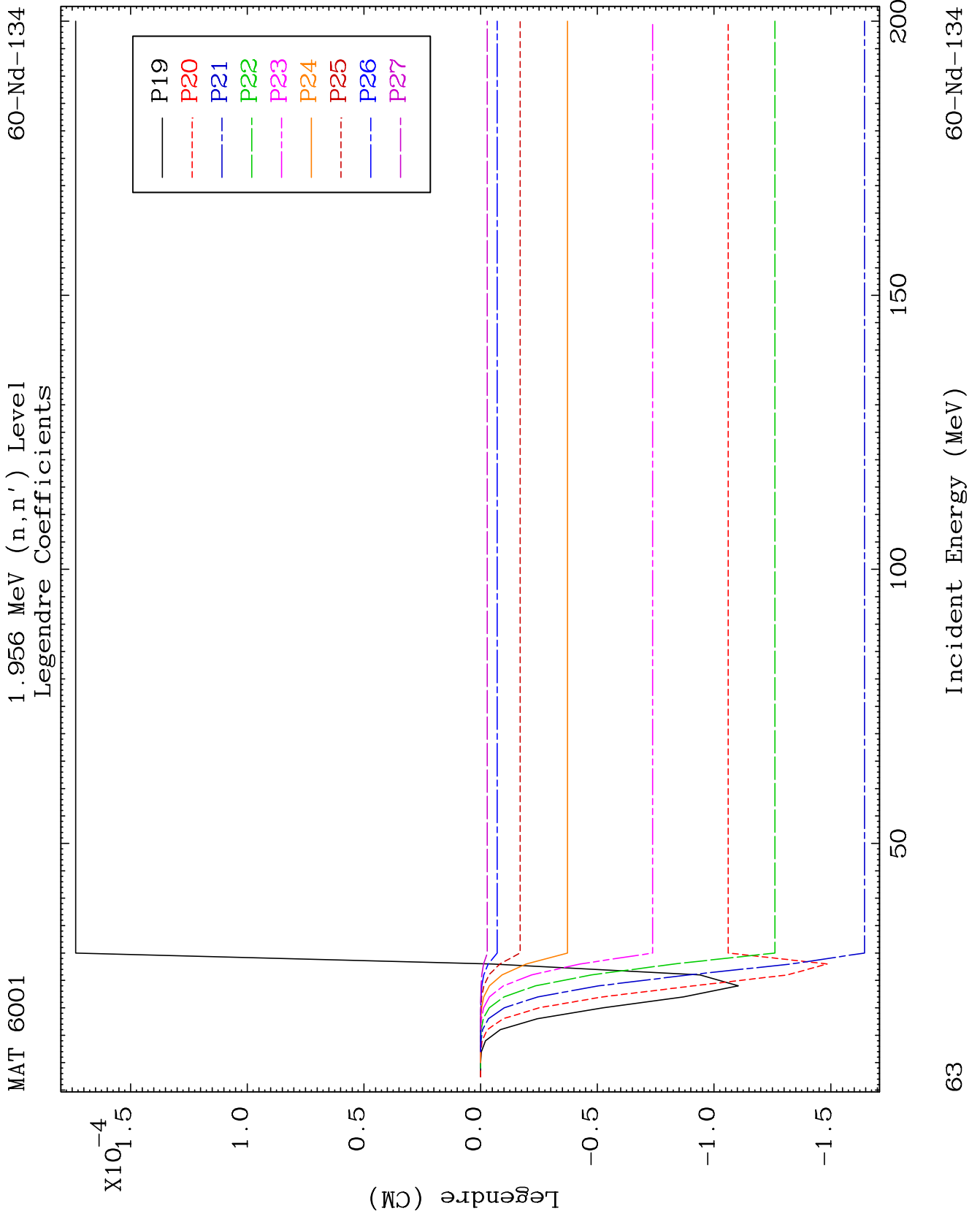


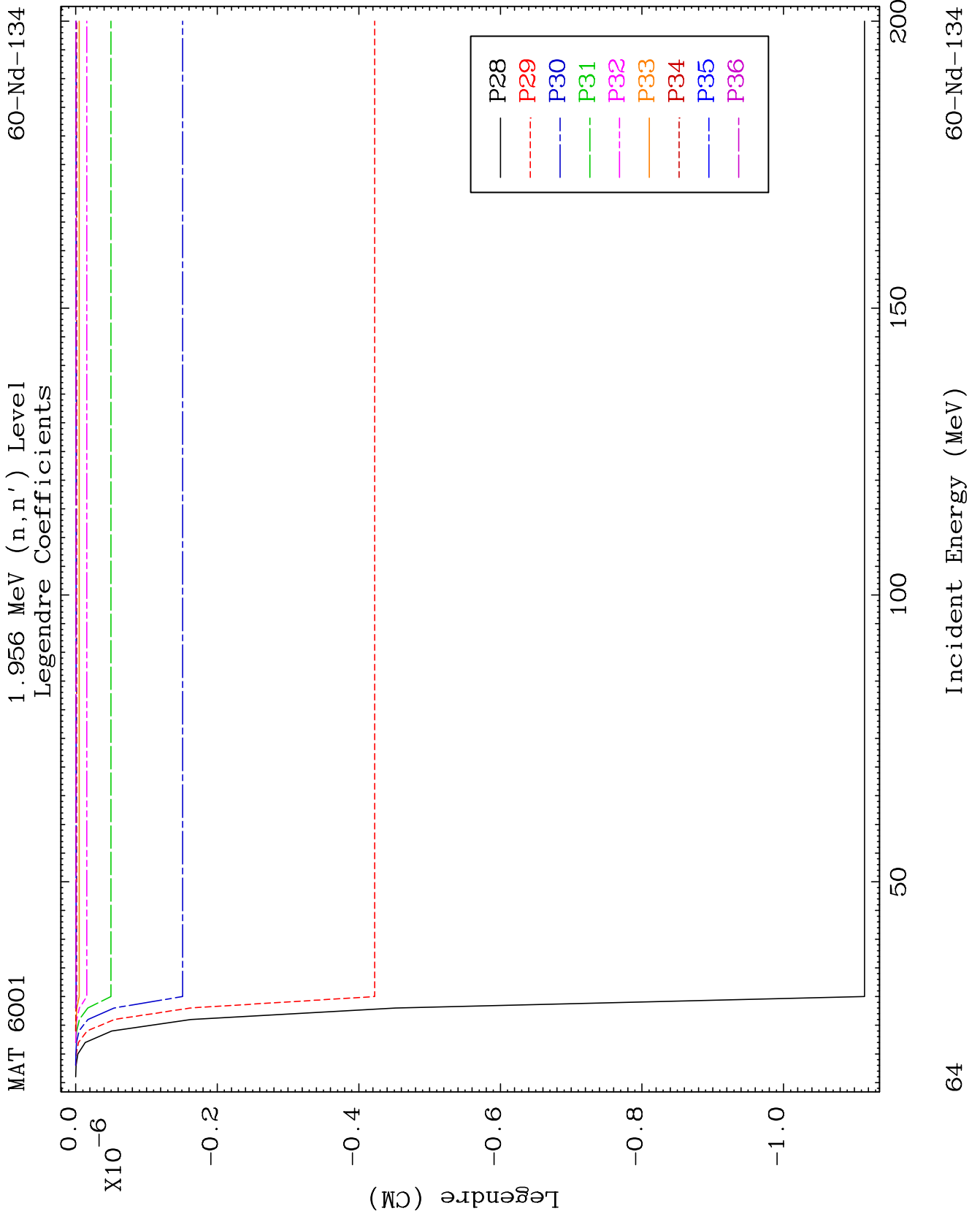


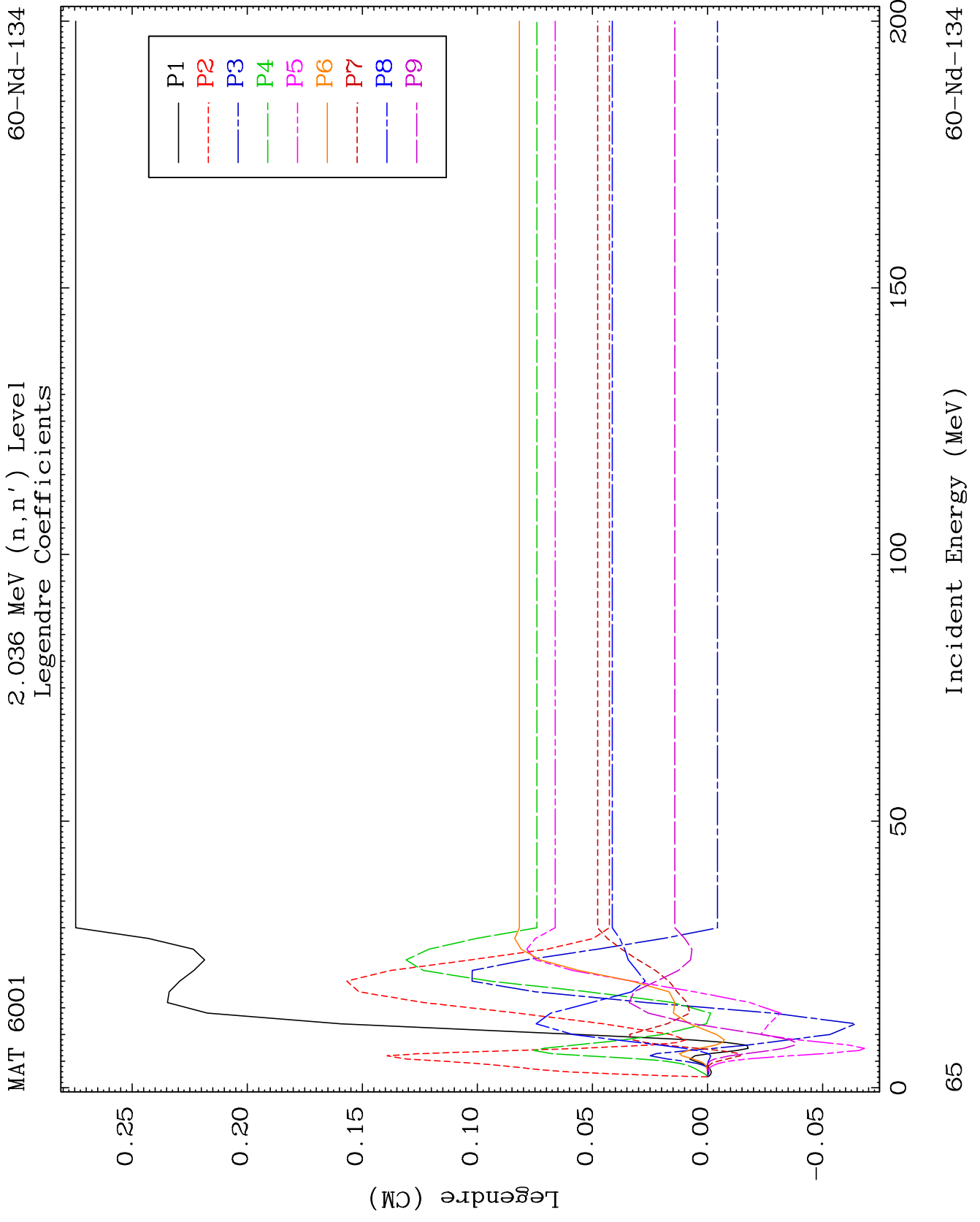


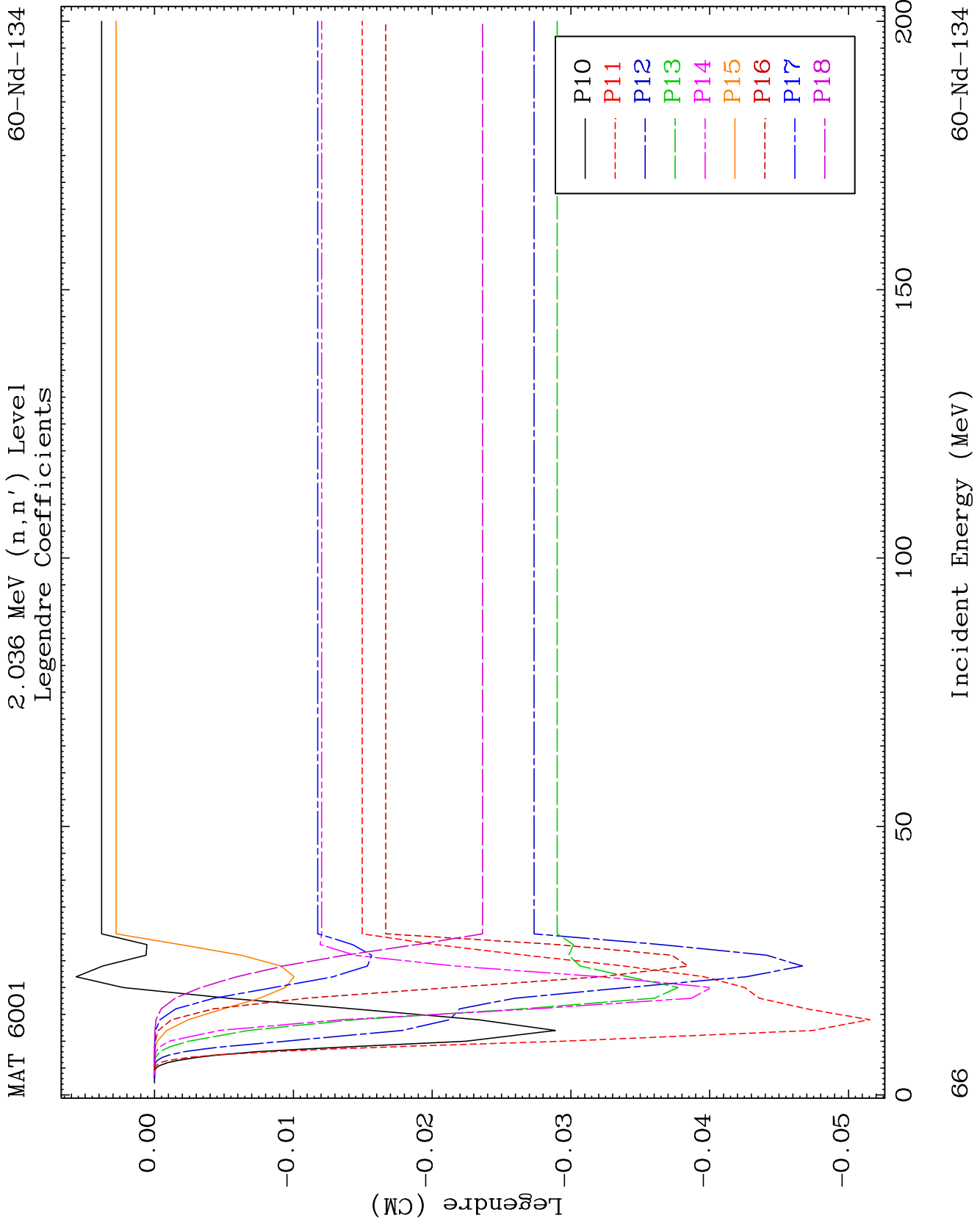


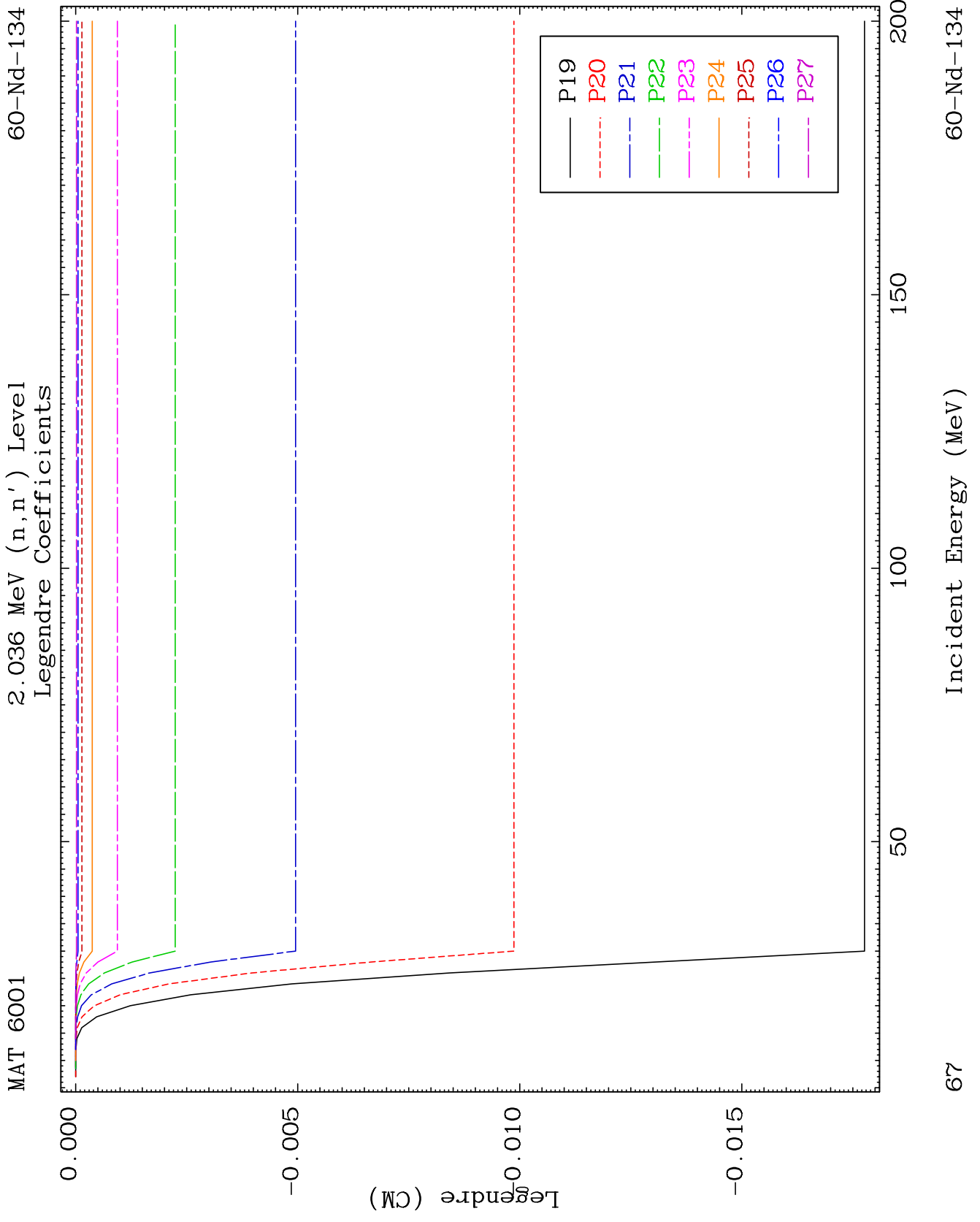


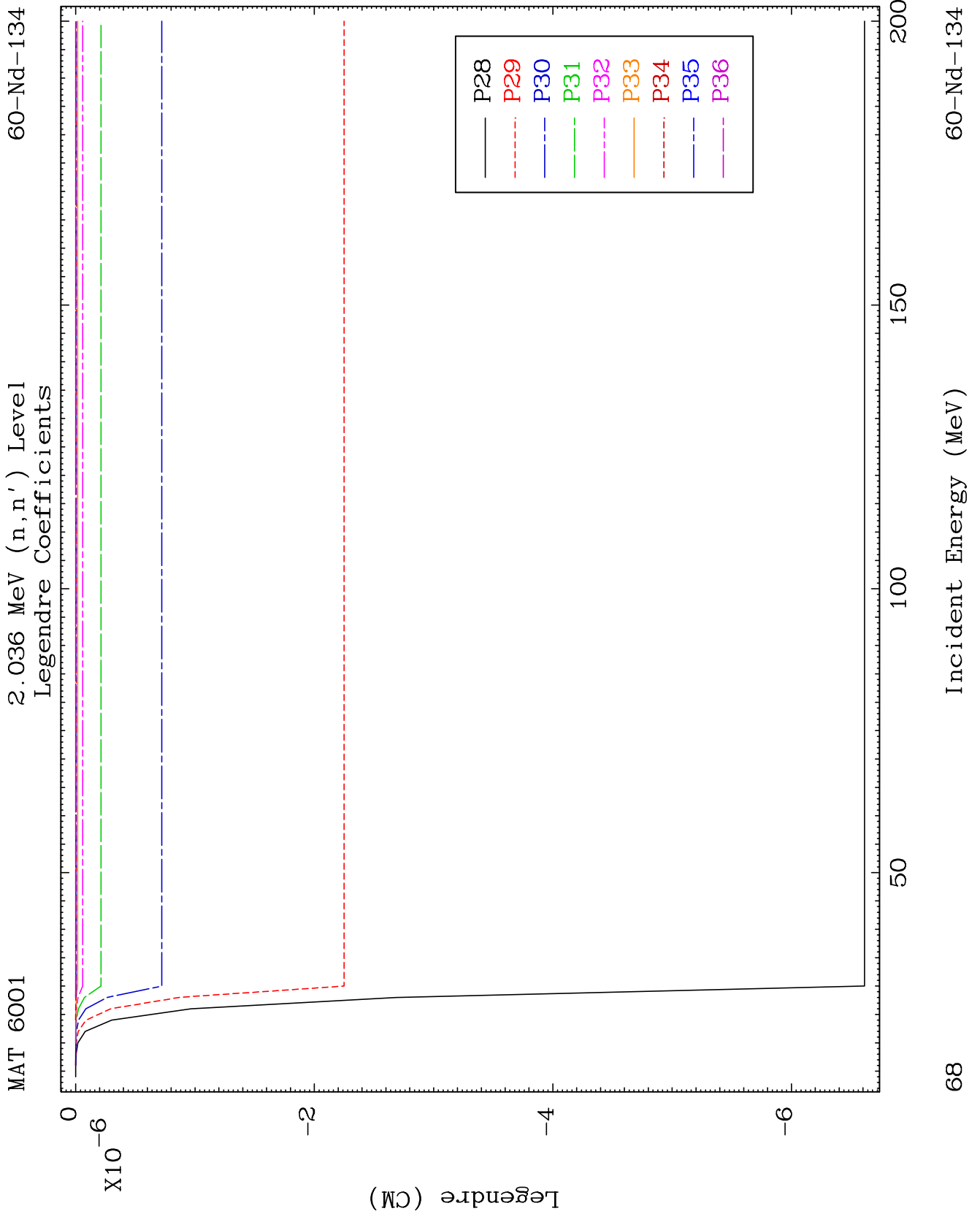


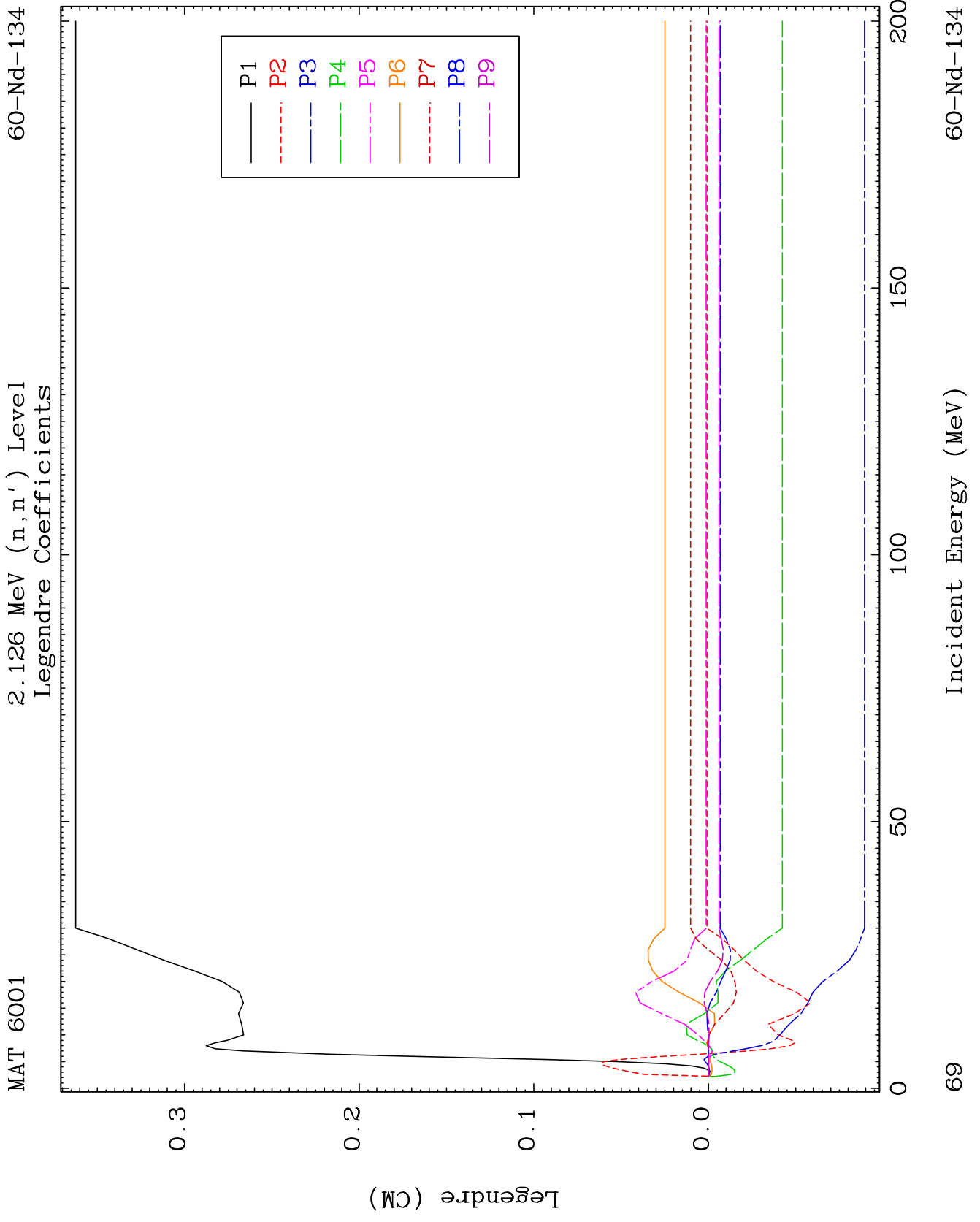


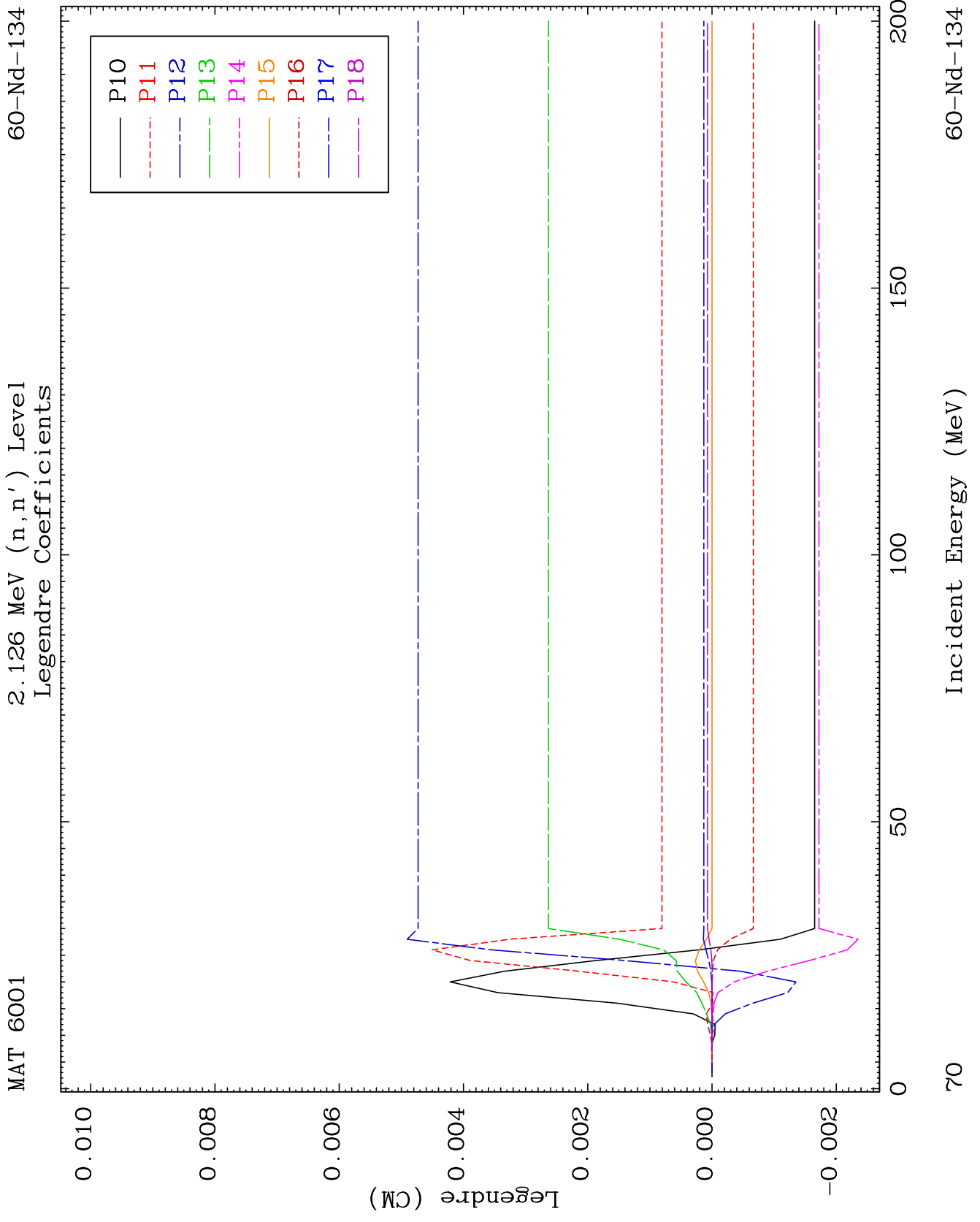


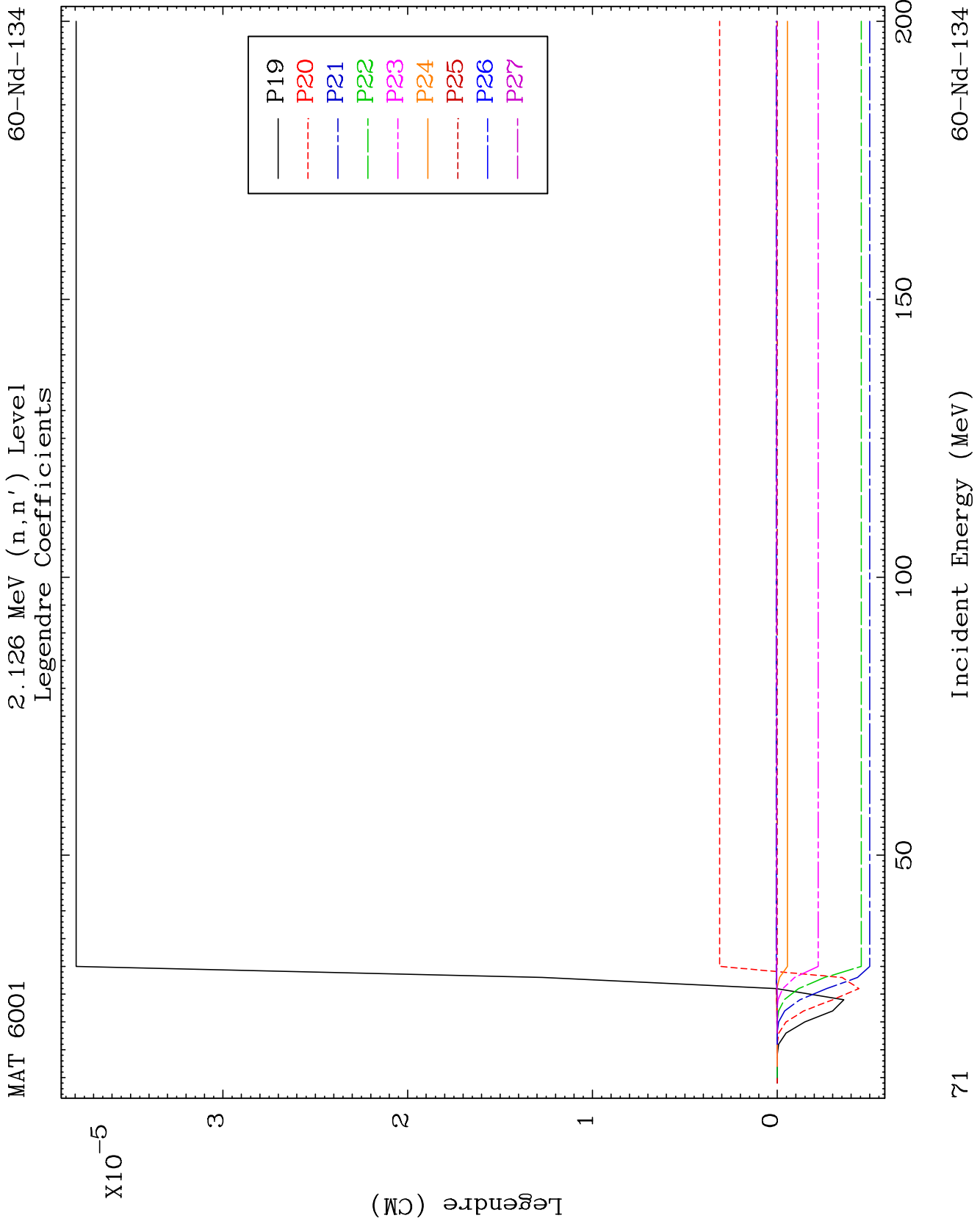


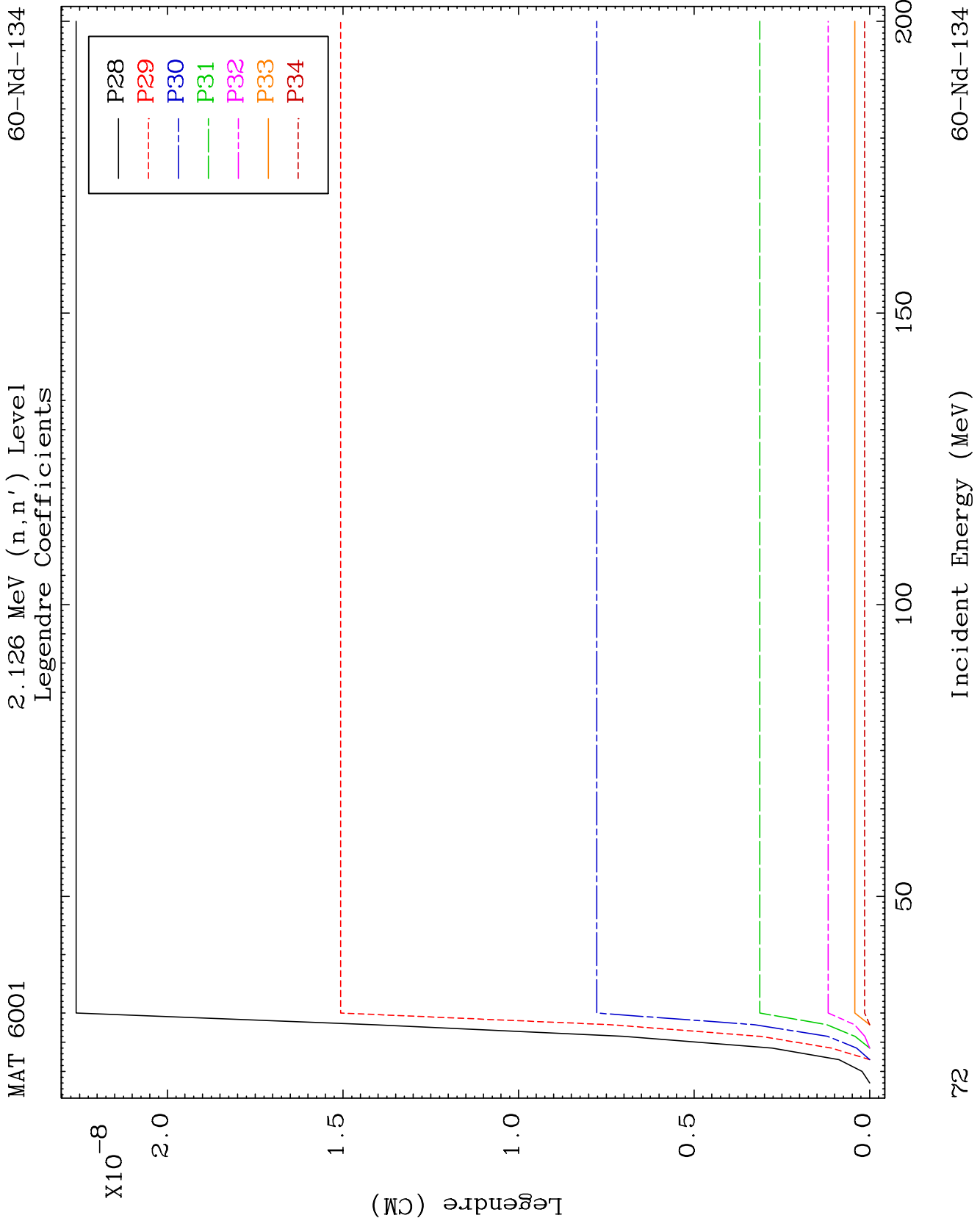


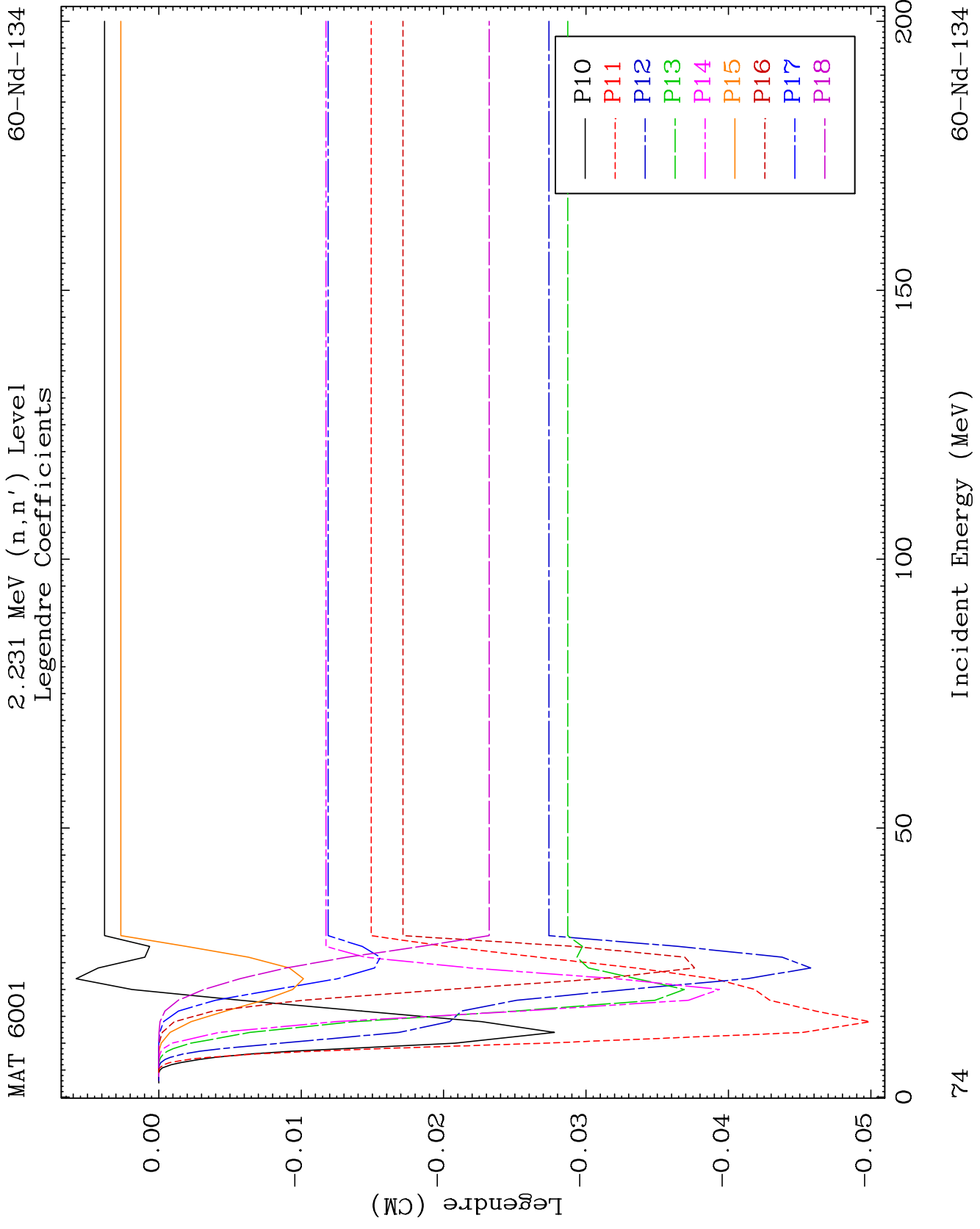


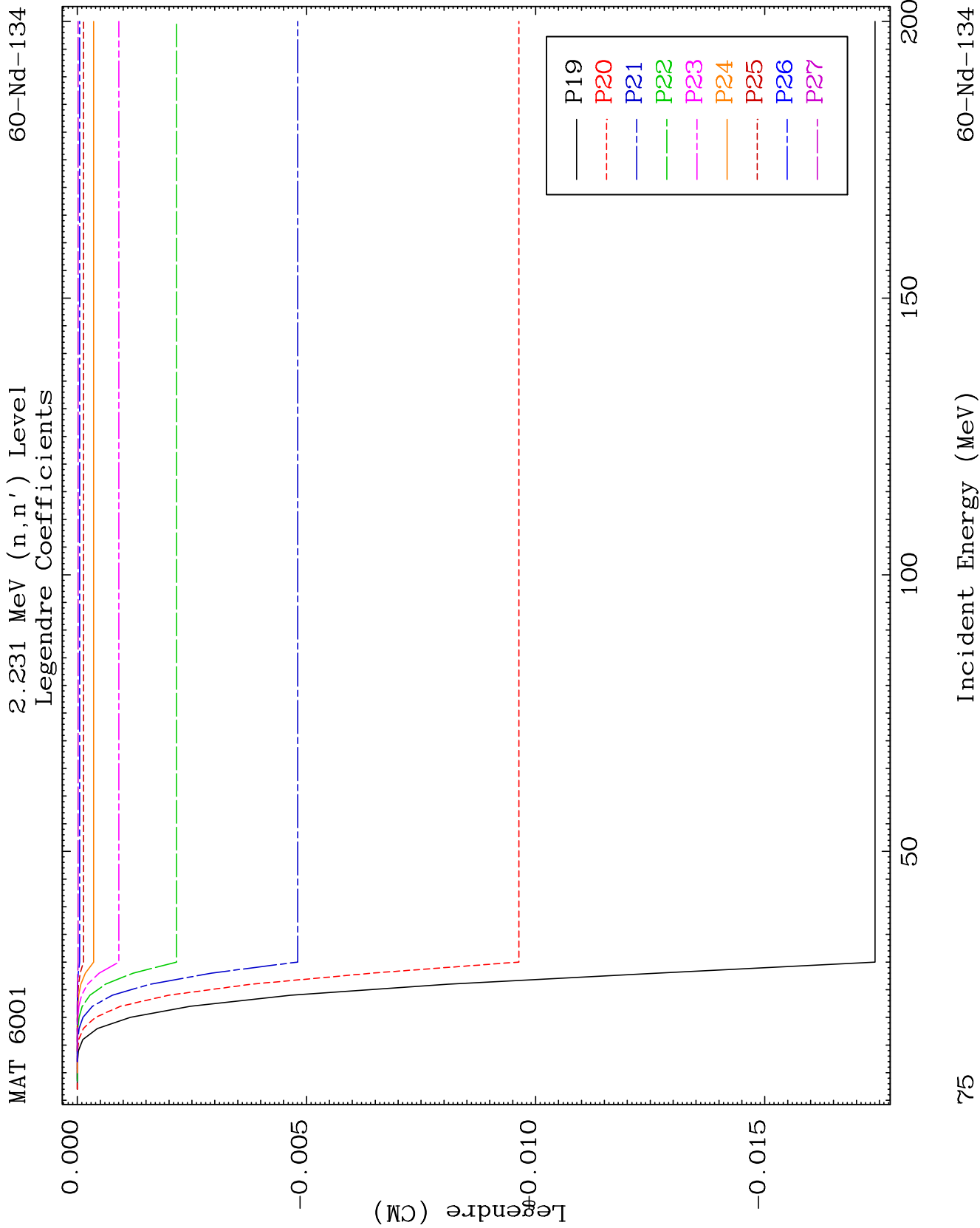


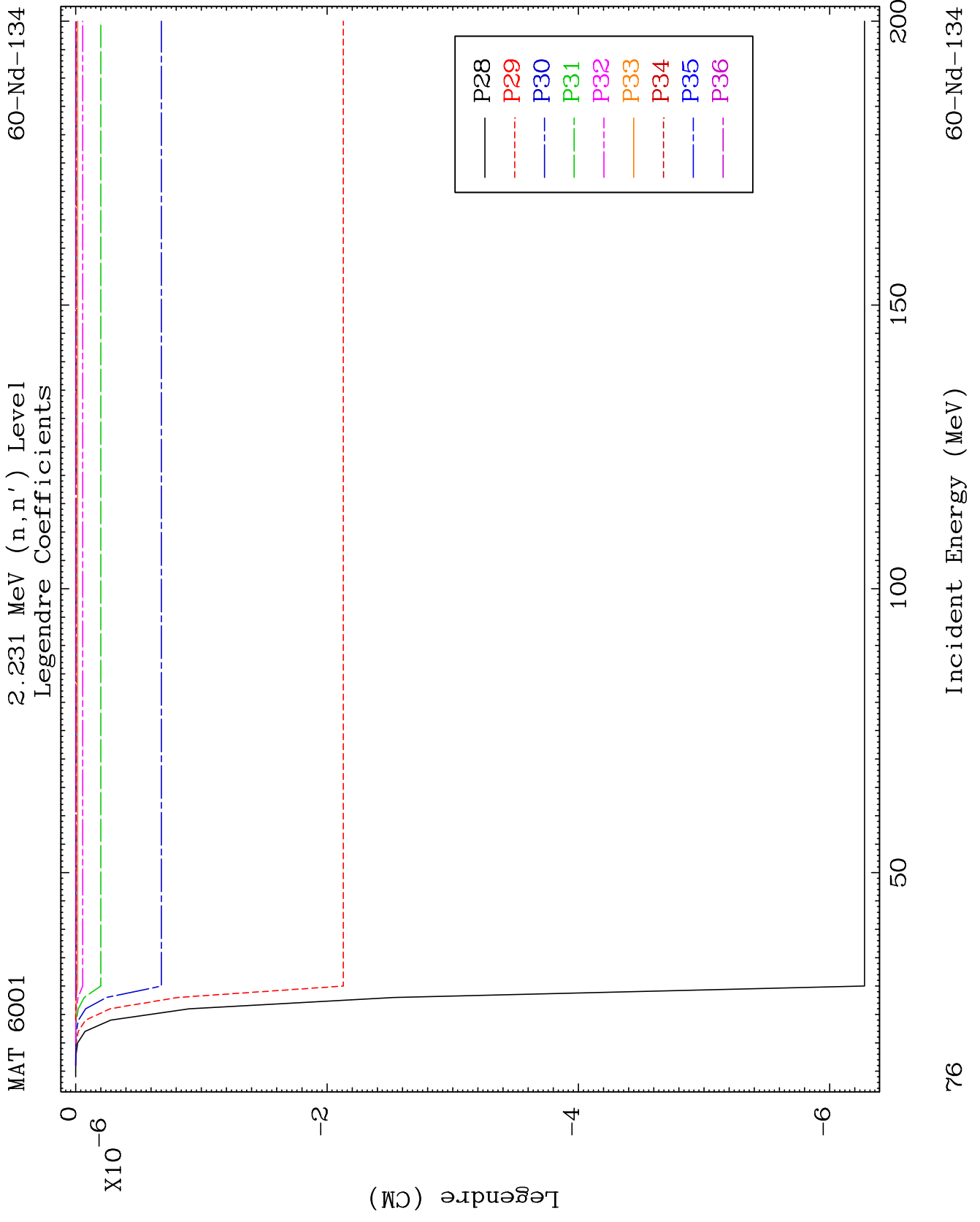










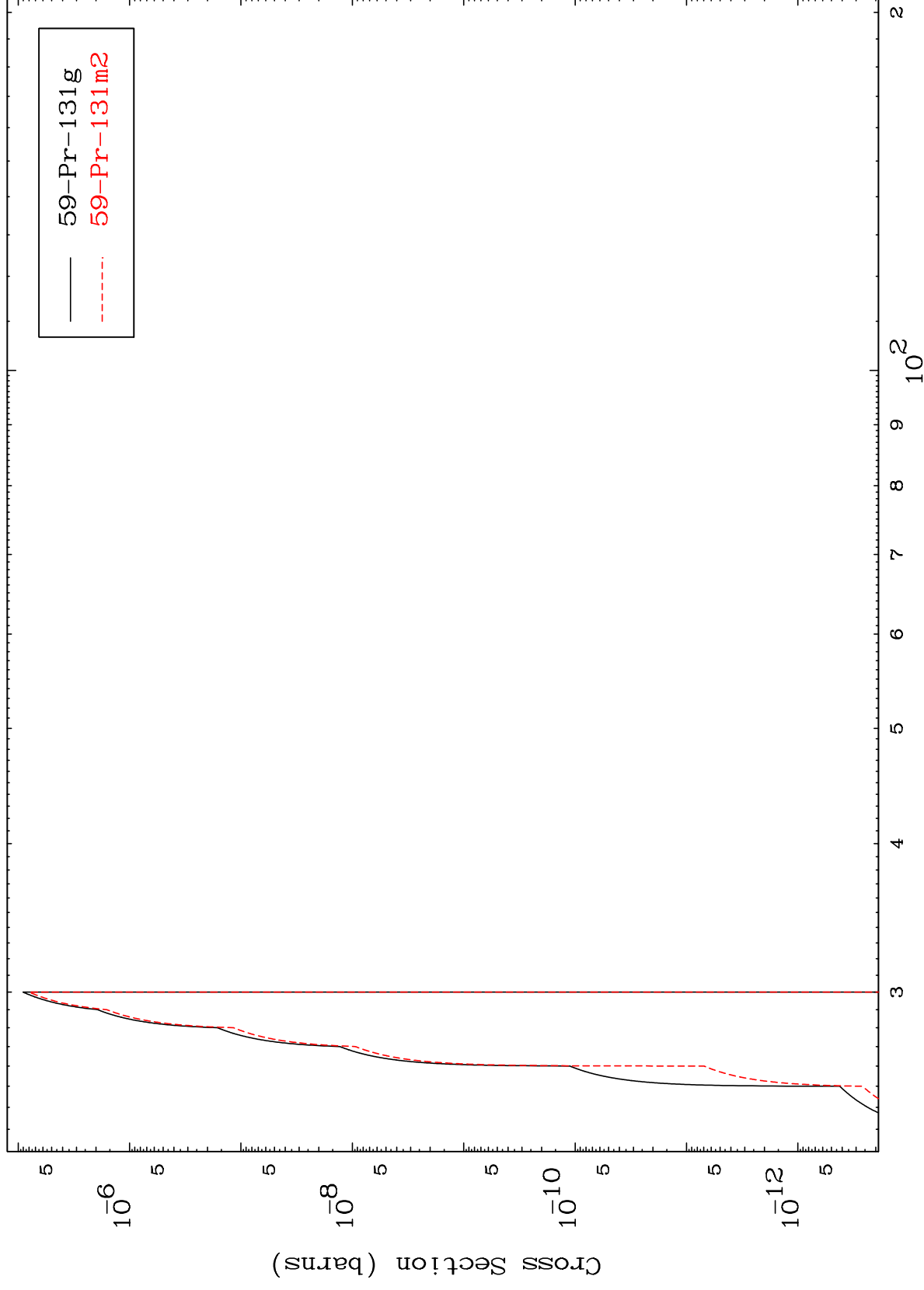


MAT 6001

(n,2n) d

60-Nd-134

Radionuclide Production Cross Section



77

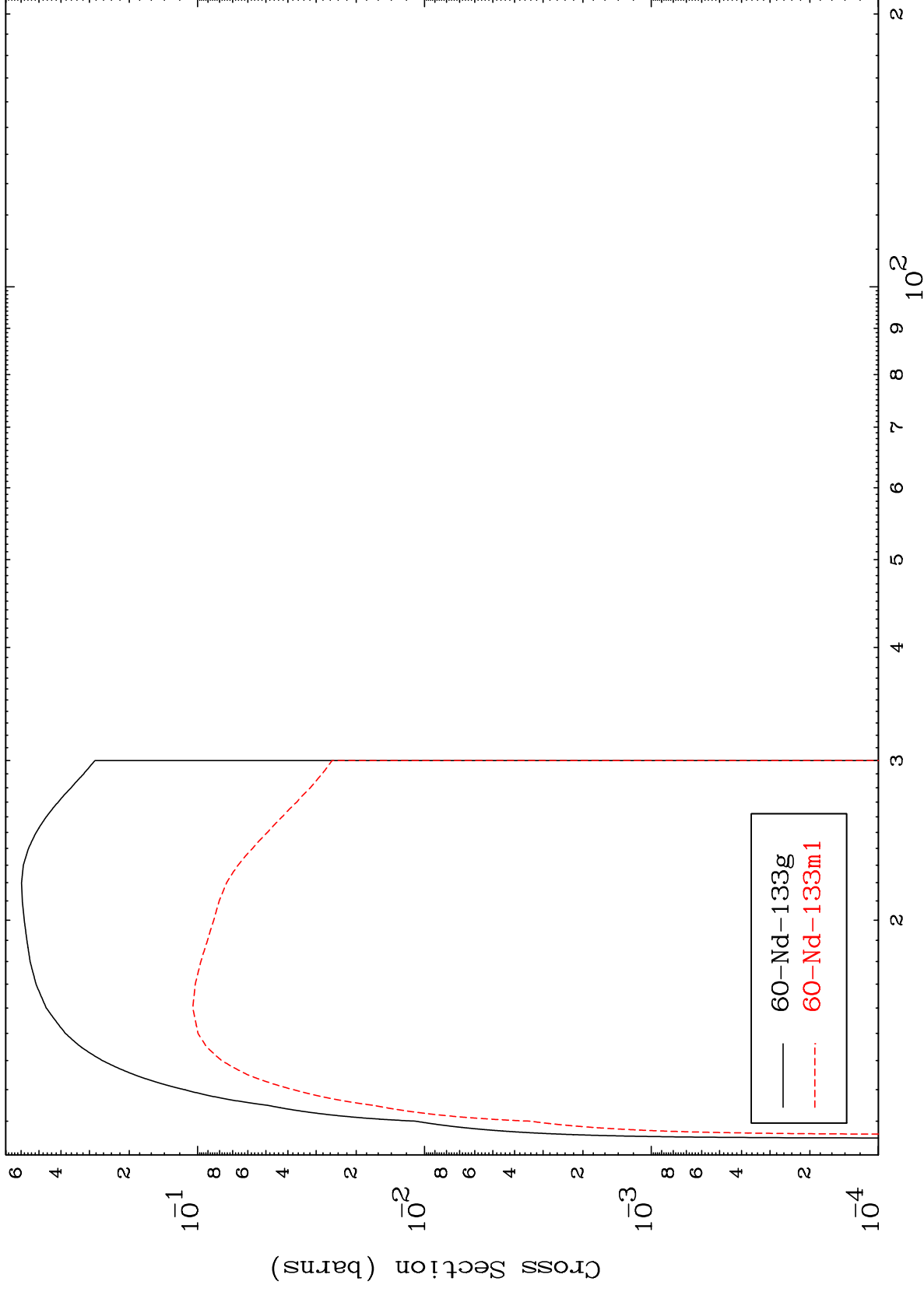
Incident Energy (MeV)

60-Nd-134

MAT 6001

60-Nd-134

(n,2n)
Radionuclide Production Cross Section



78

Incident Energy (MeV)

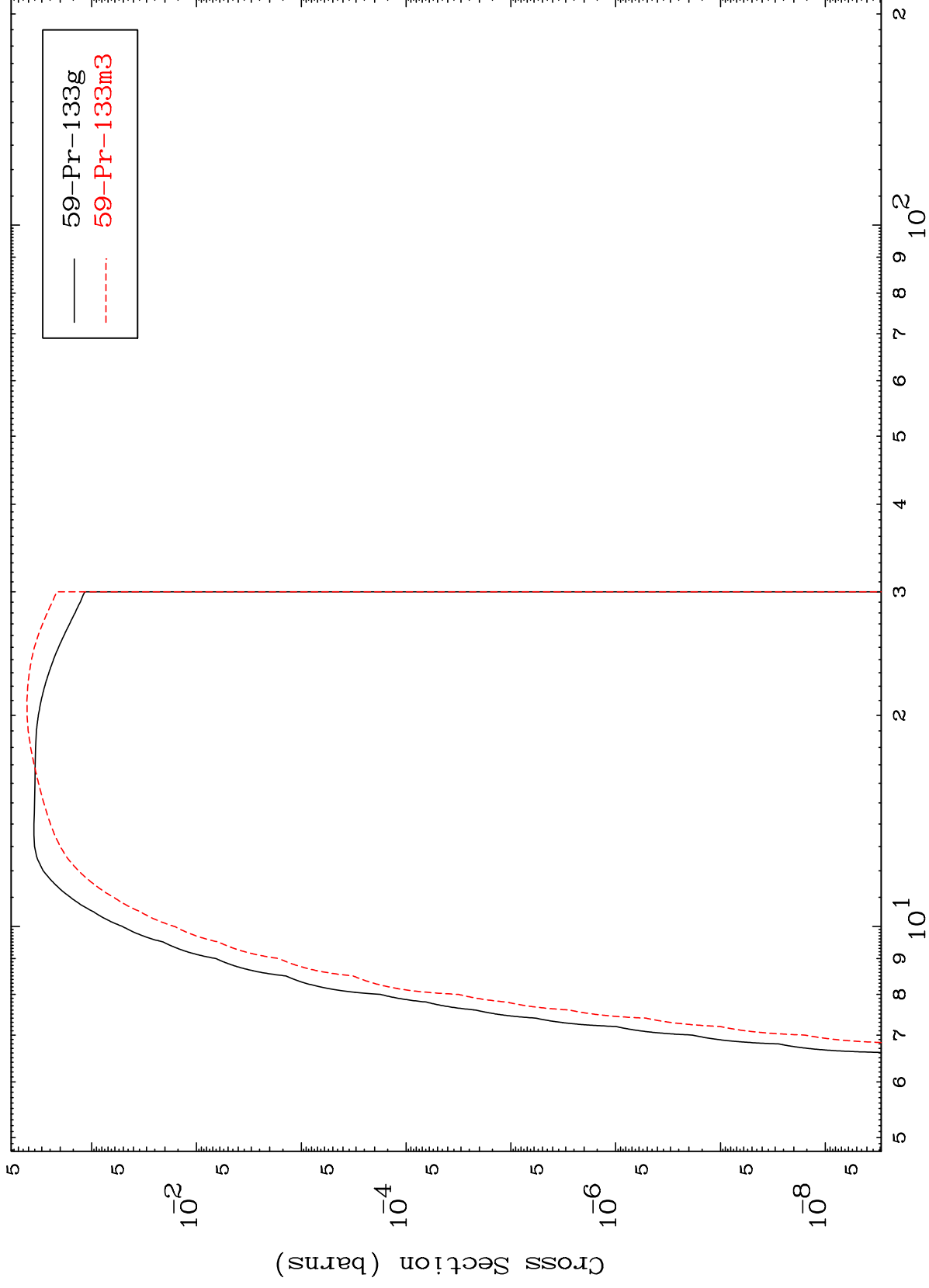
60-Nd-134

MAT 6001

(n,n') p

60-Nd-134

Radionuclide Production Cross Section



79

Incident Energy (MeV)

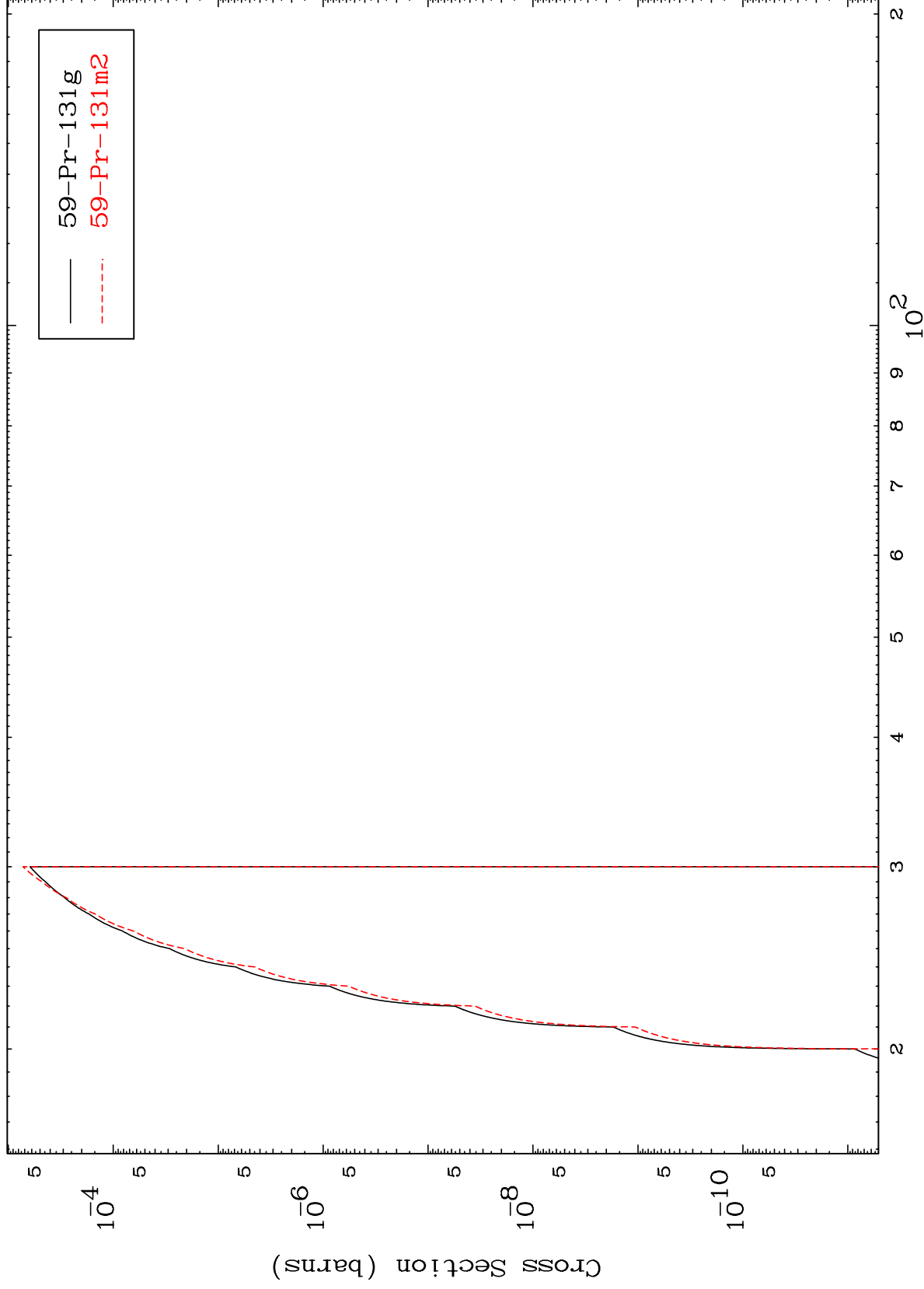
60-Nd-134

MAT 6001

(n,n') t

60-Nd-134

Radionuclide Production Cross Section



80

Incident Energy (MeV)

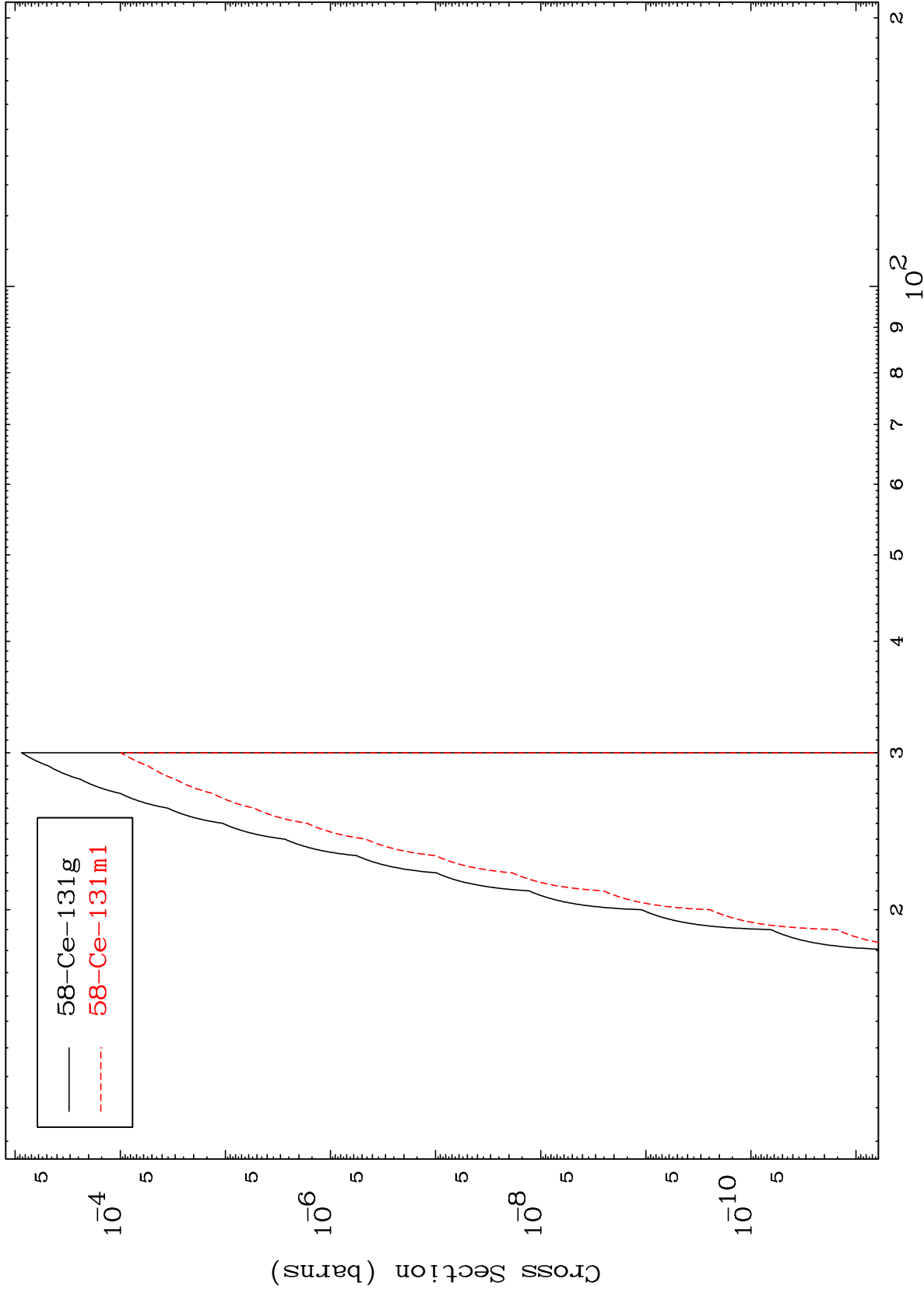
60-Nd-134

MAT 6001

(n,n') He-3

60-Nd-134

Radionuclide Production Cross Section



81

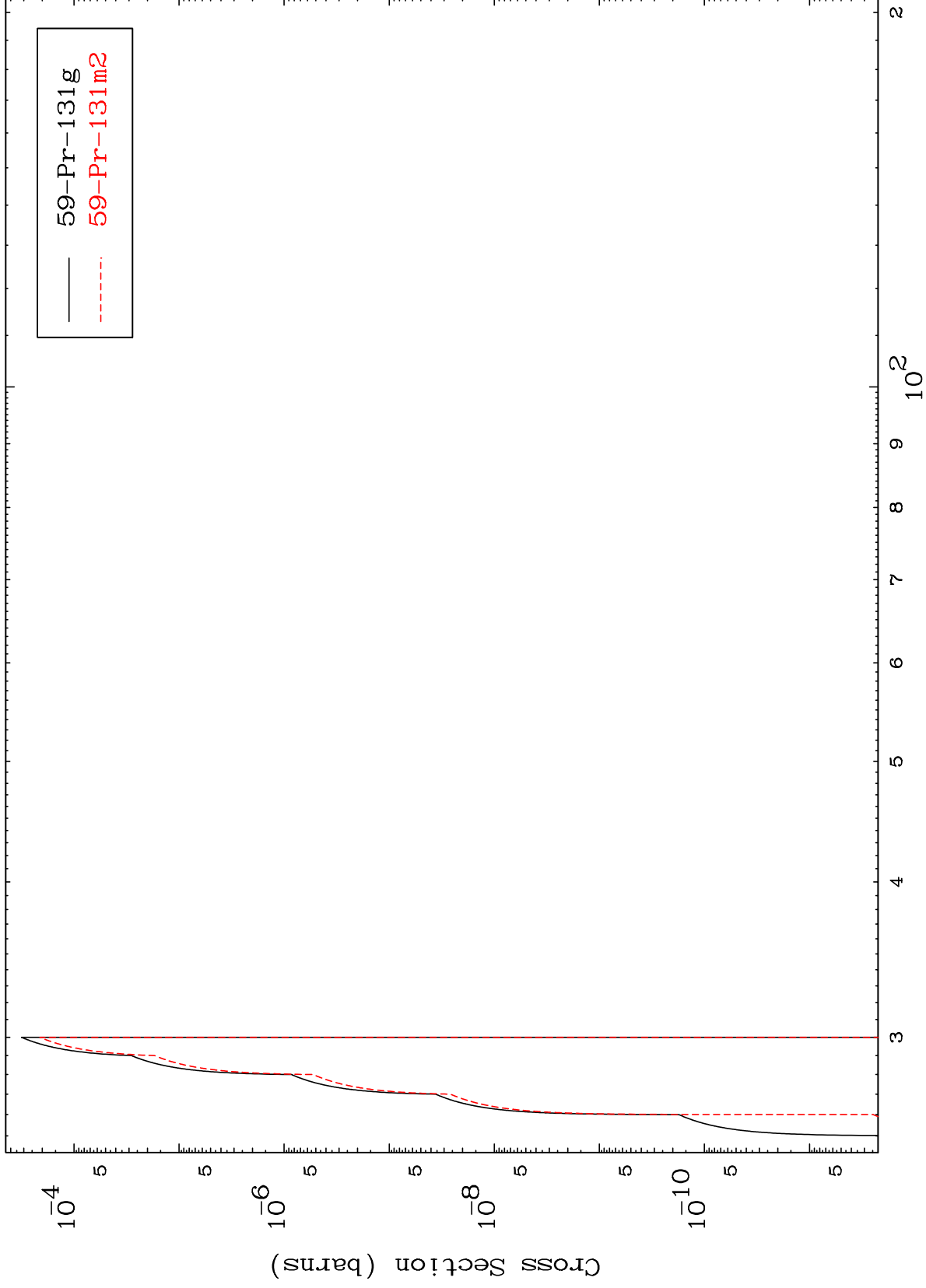
Incident Energy (MeV)

60-Nd-134

MAT 6001

60-Nd-134

(n,3n) p
Radionuclide Production Cross Section



82

60-Nd-134

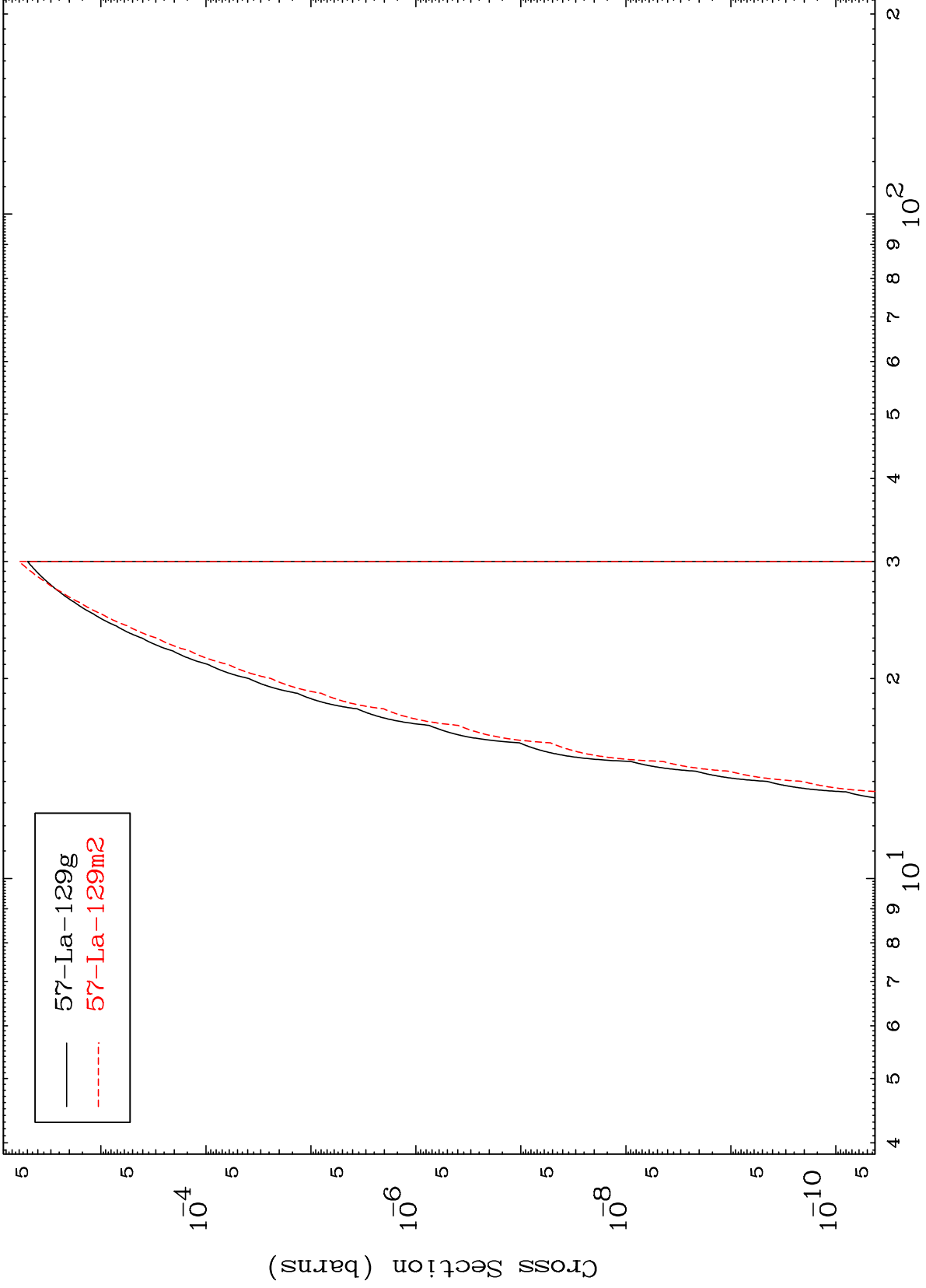
Incident Energy (MeV)

MAT 6001

(n,n') p α

60-Nd-134

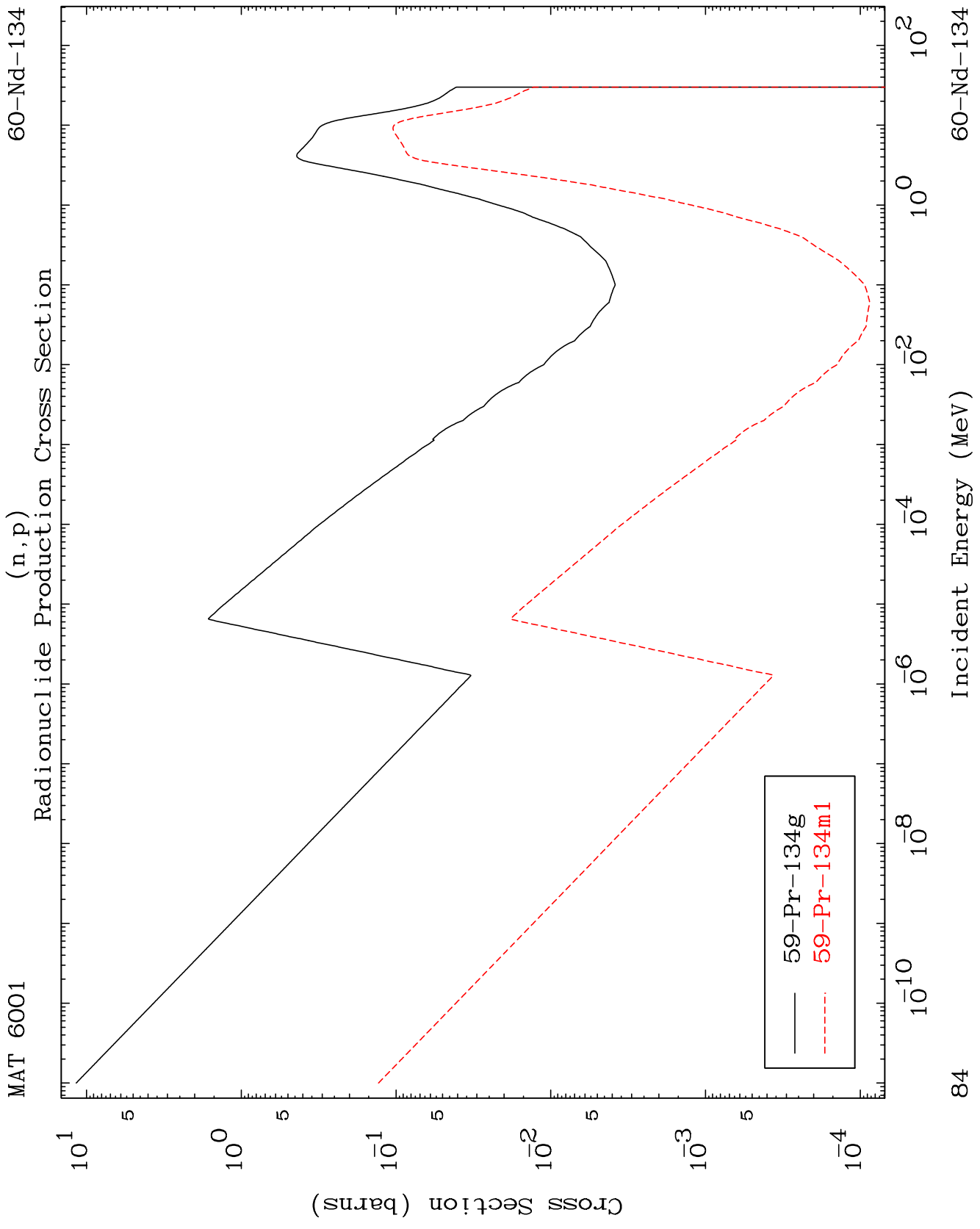
Radionuclide Production Cross Section



83

Incident Energy (MeV)

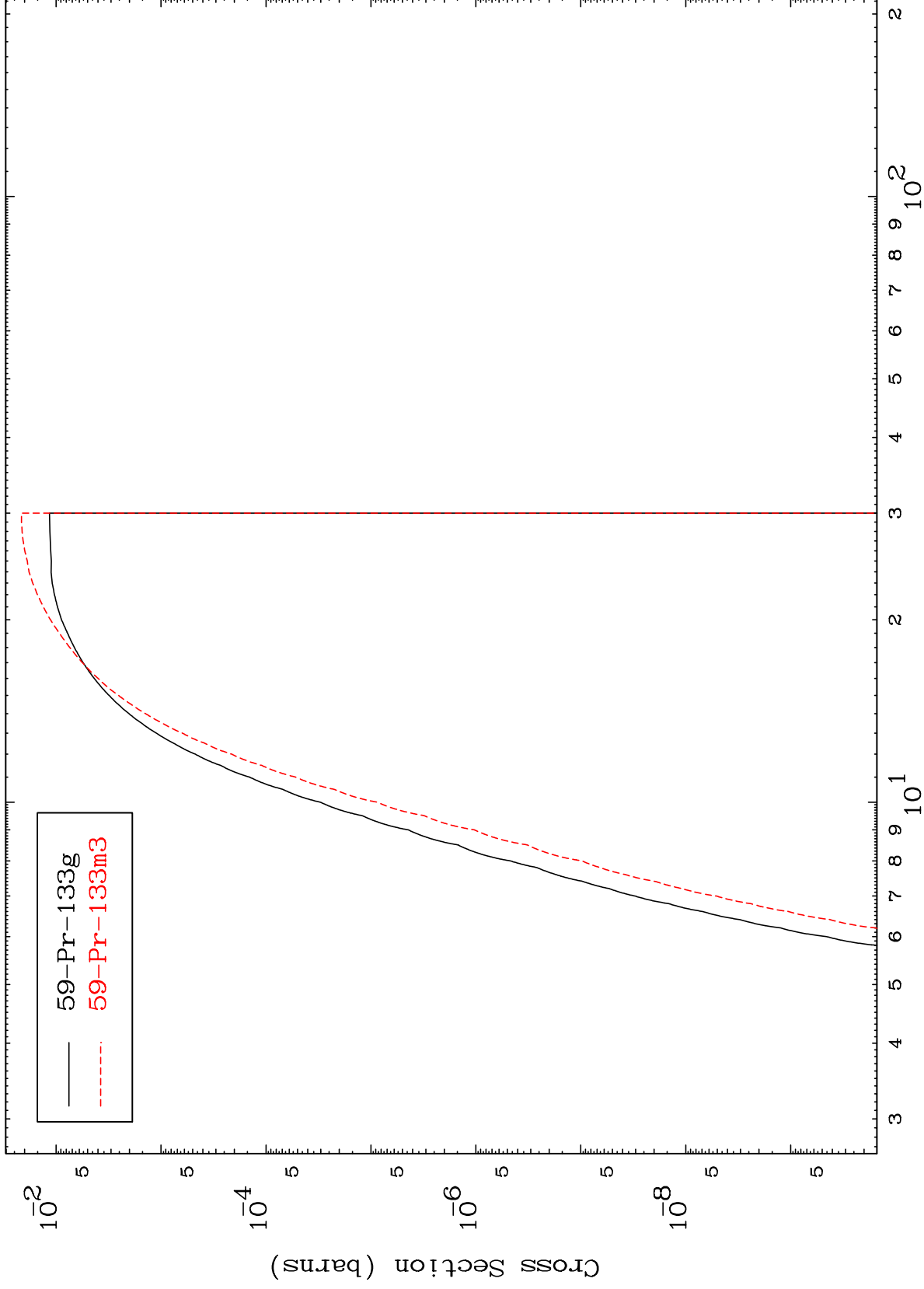
60-Nd-134



MAT 6001

$^{60}\text{Nd-134}$

(n,d)
Radionuclide Production Cross Section



85

Incident Energy (MeV)

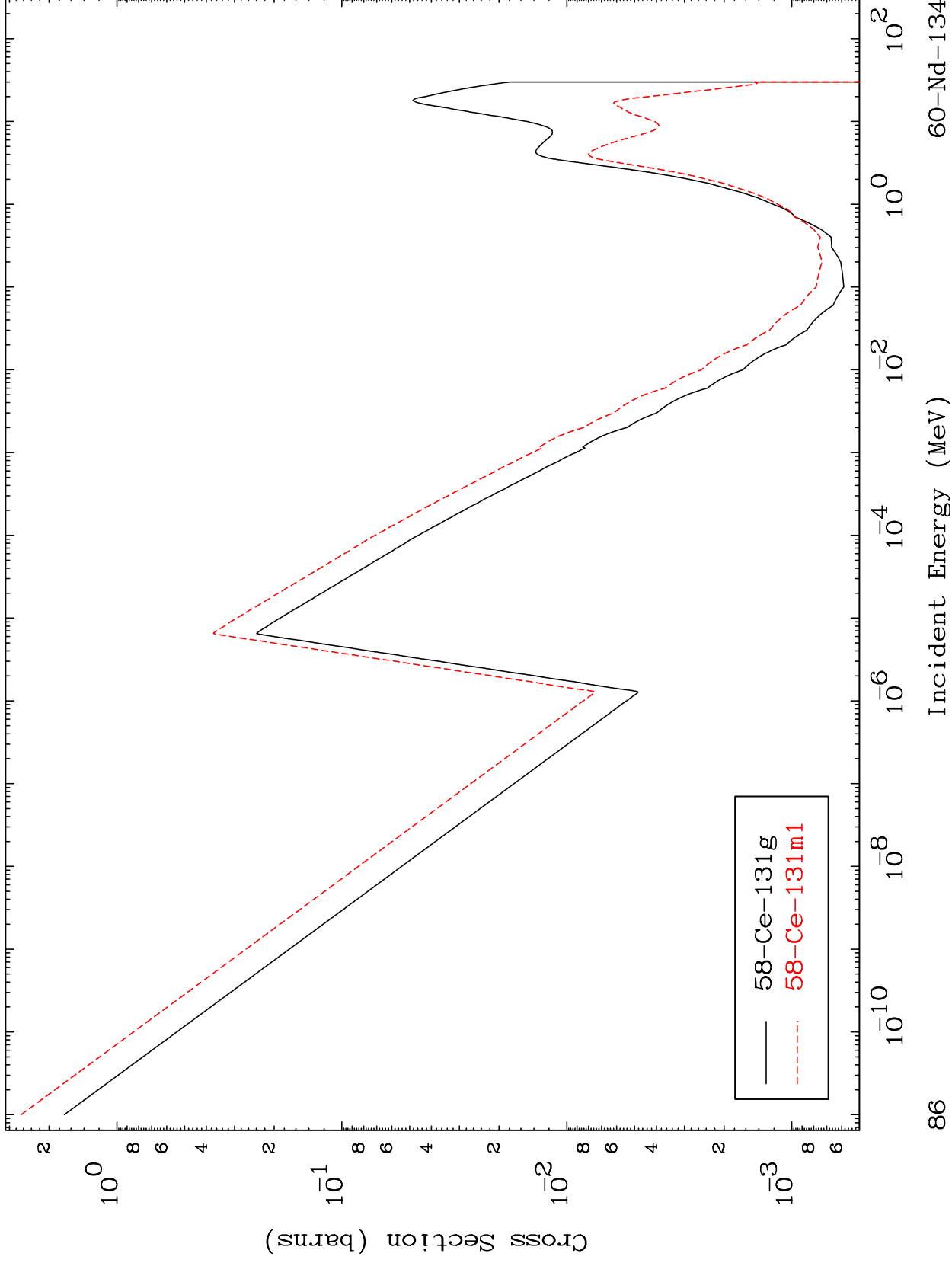
$^{60}\text{Nd-134}$

MAT 6001

(n, α)

60-Nd-134

Radionuclide Production Cross Section

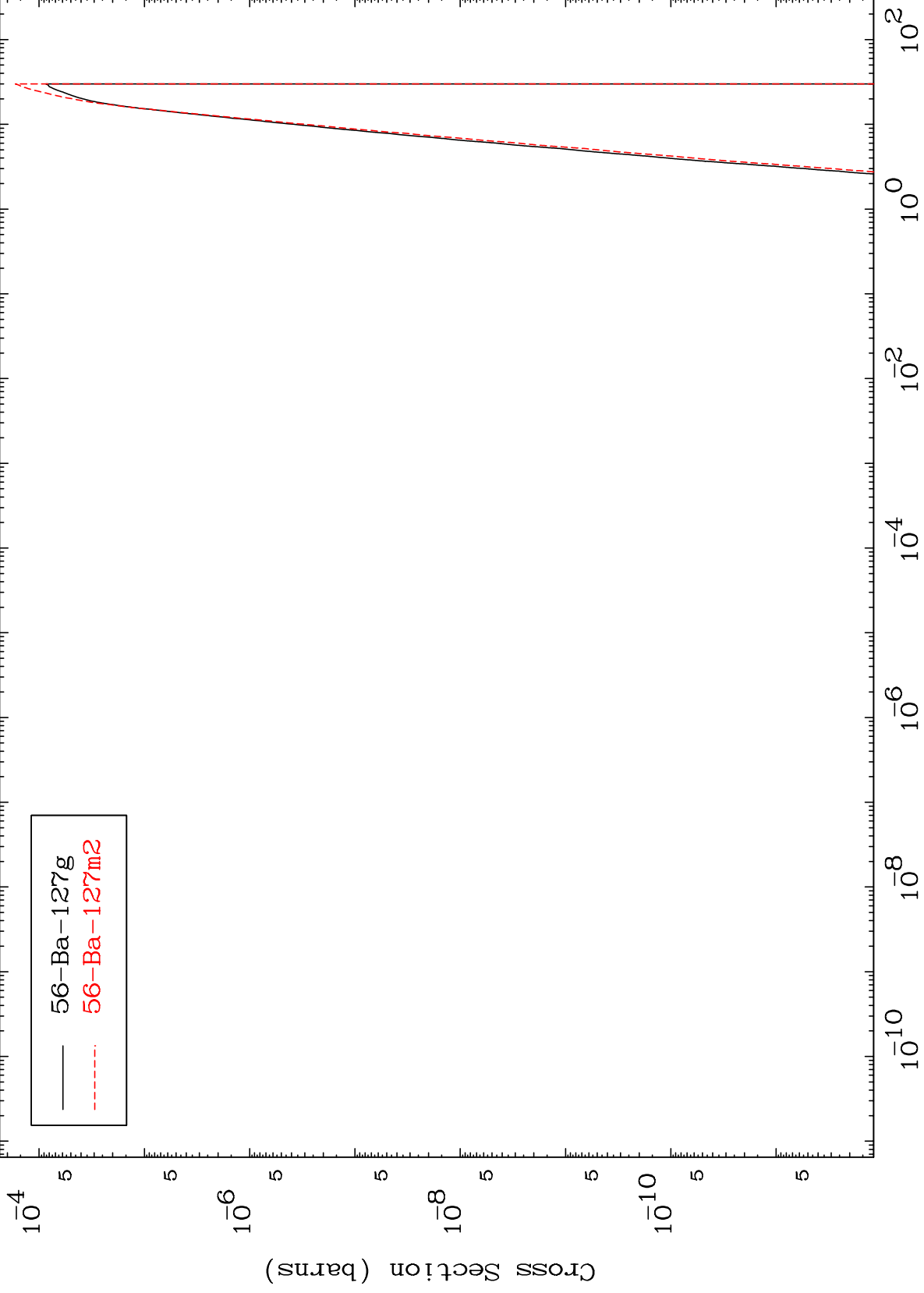


MAT 6001

(n,2α)

60-Nd-134

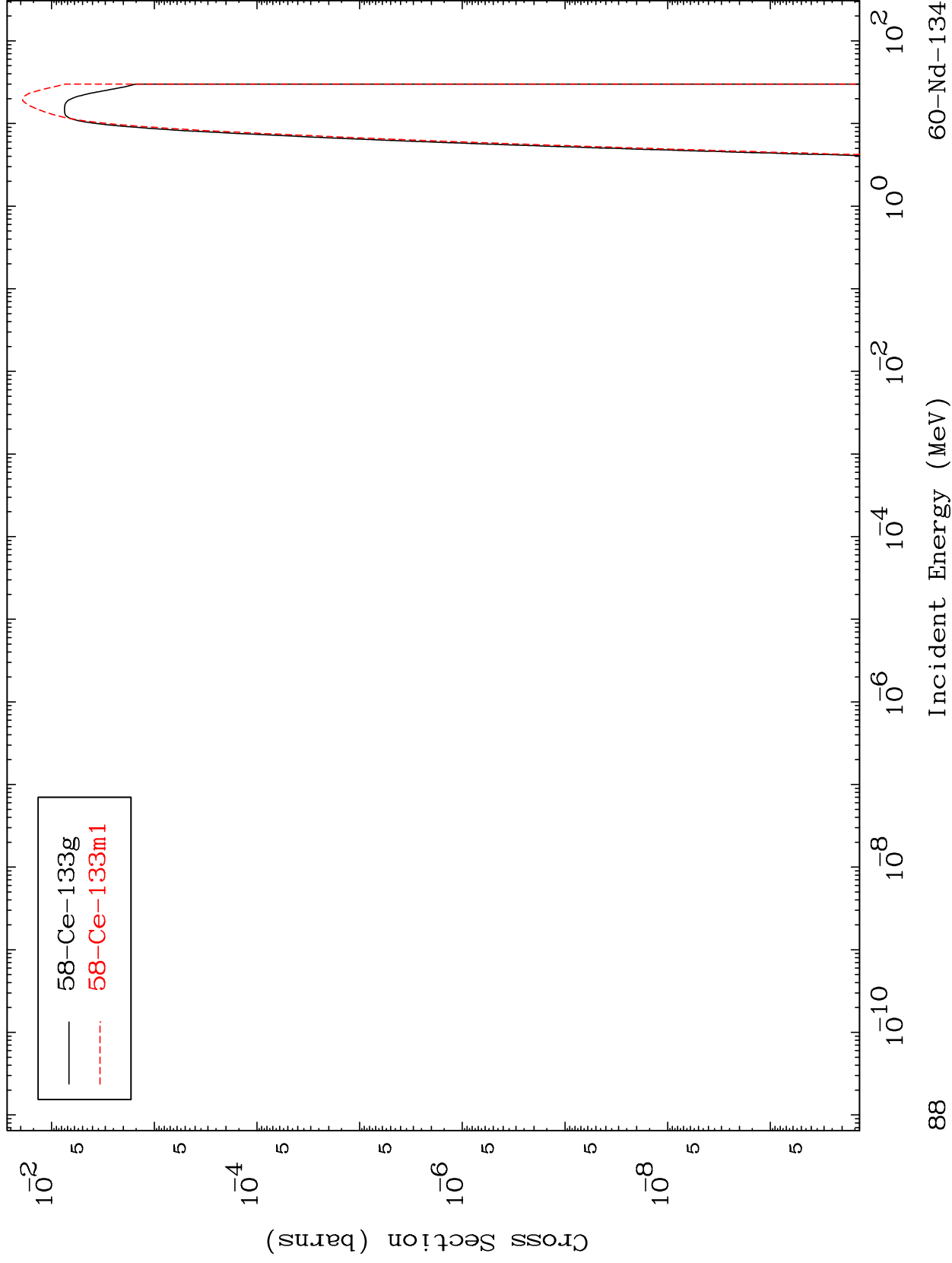
Radionuclide Production Cross Section



MAT 6001

60-Nd-134

(n,2p)
Radionuclide Production Cross Section

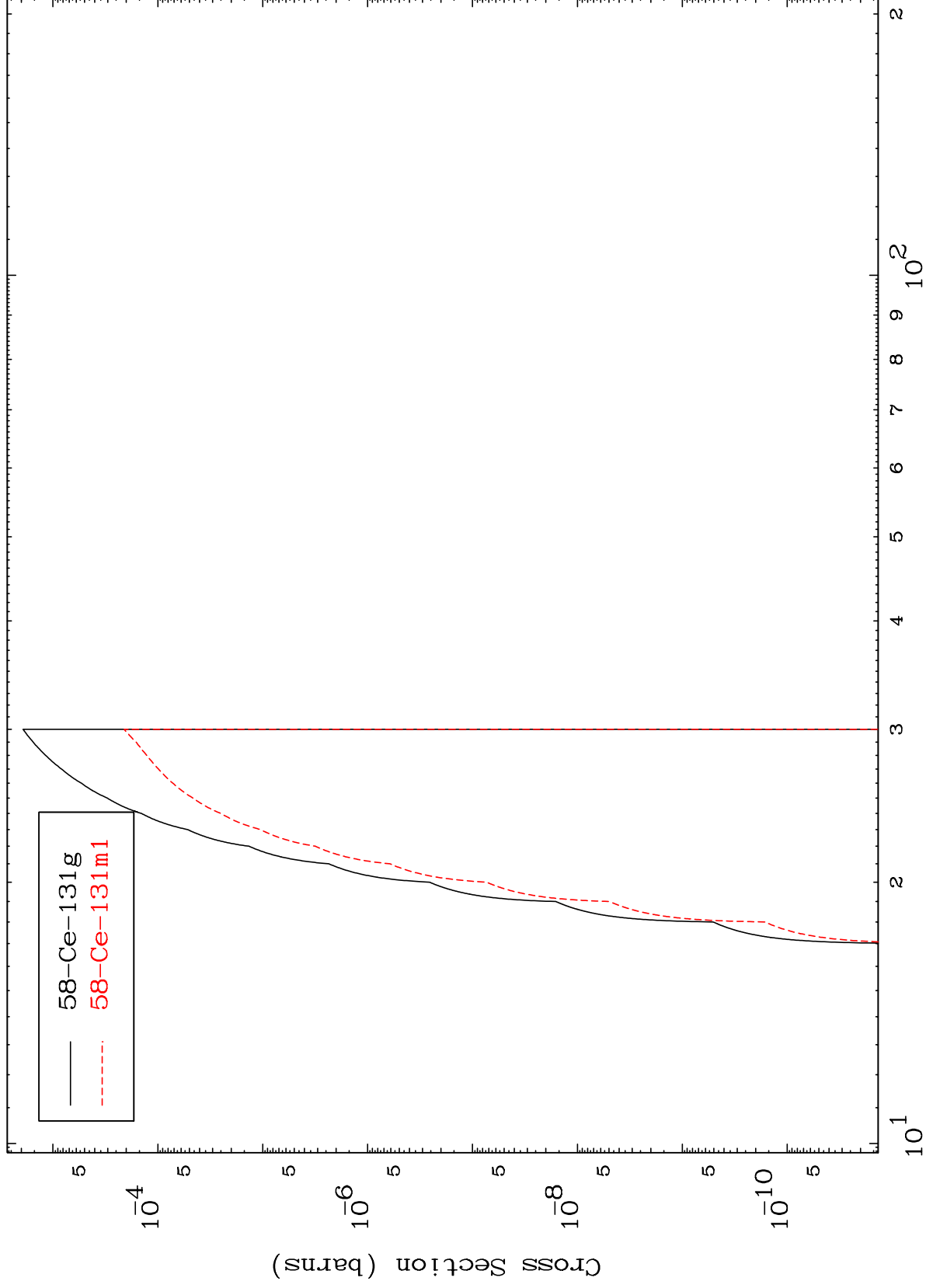


MAT 6001

(n,p) t

60-Nd-134

Radionuclide Production Cross Section



Incident Energy (MeV)

60-Nd-134

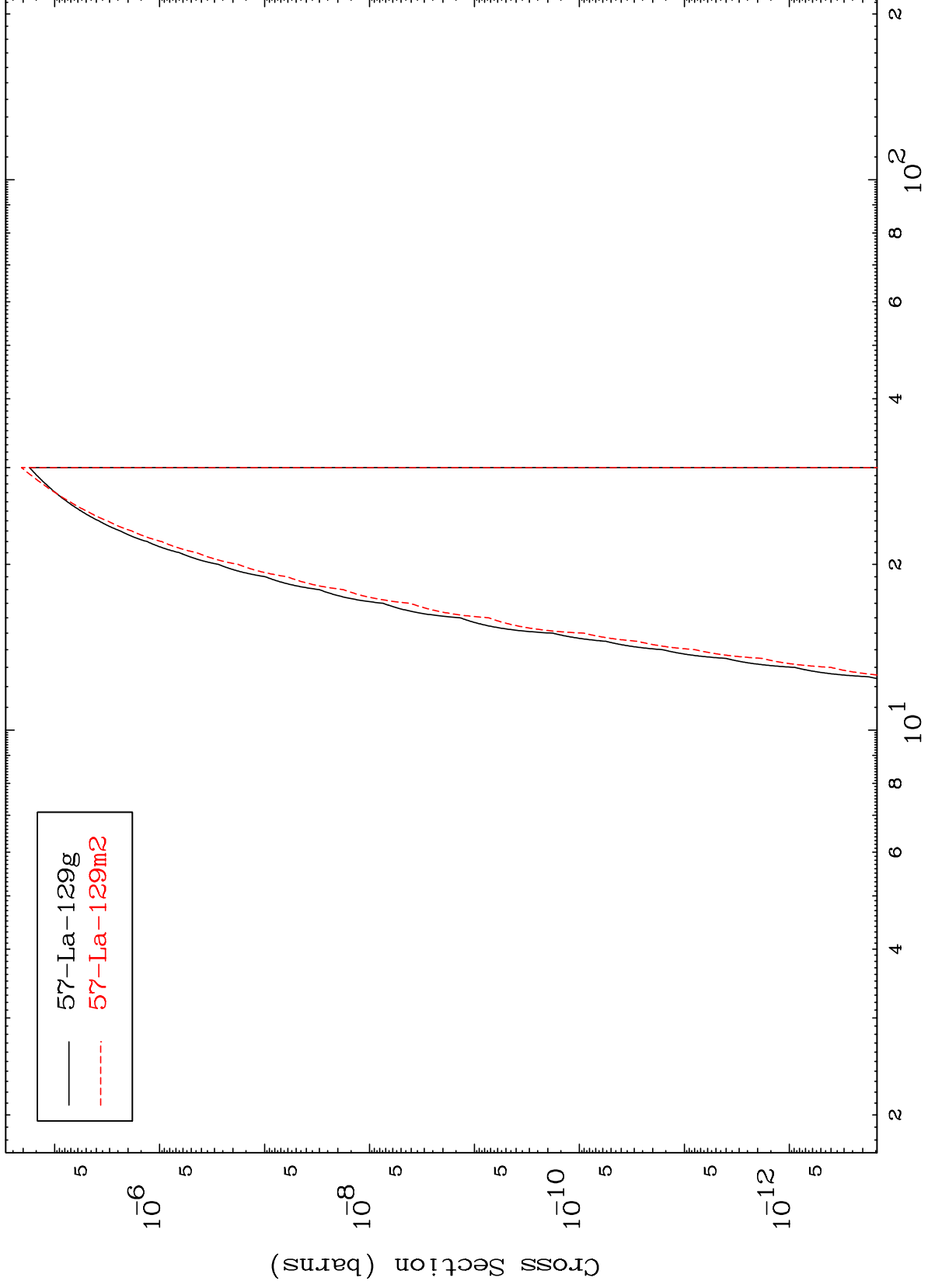
89

MAT 6001

(n,d) α

$^{60}\text{Nd-134}$

Radionuclide Production Cross Section



90

Incident Energy (MeV)

$^{60}\text{Nd-134}$