

Program EVALPLOT
(Version 2015-2)

by

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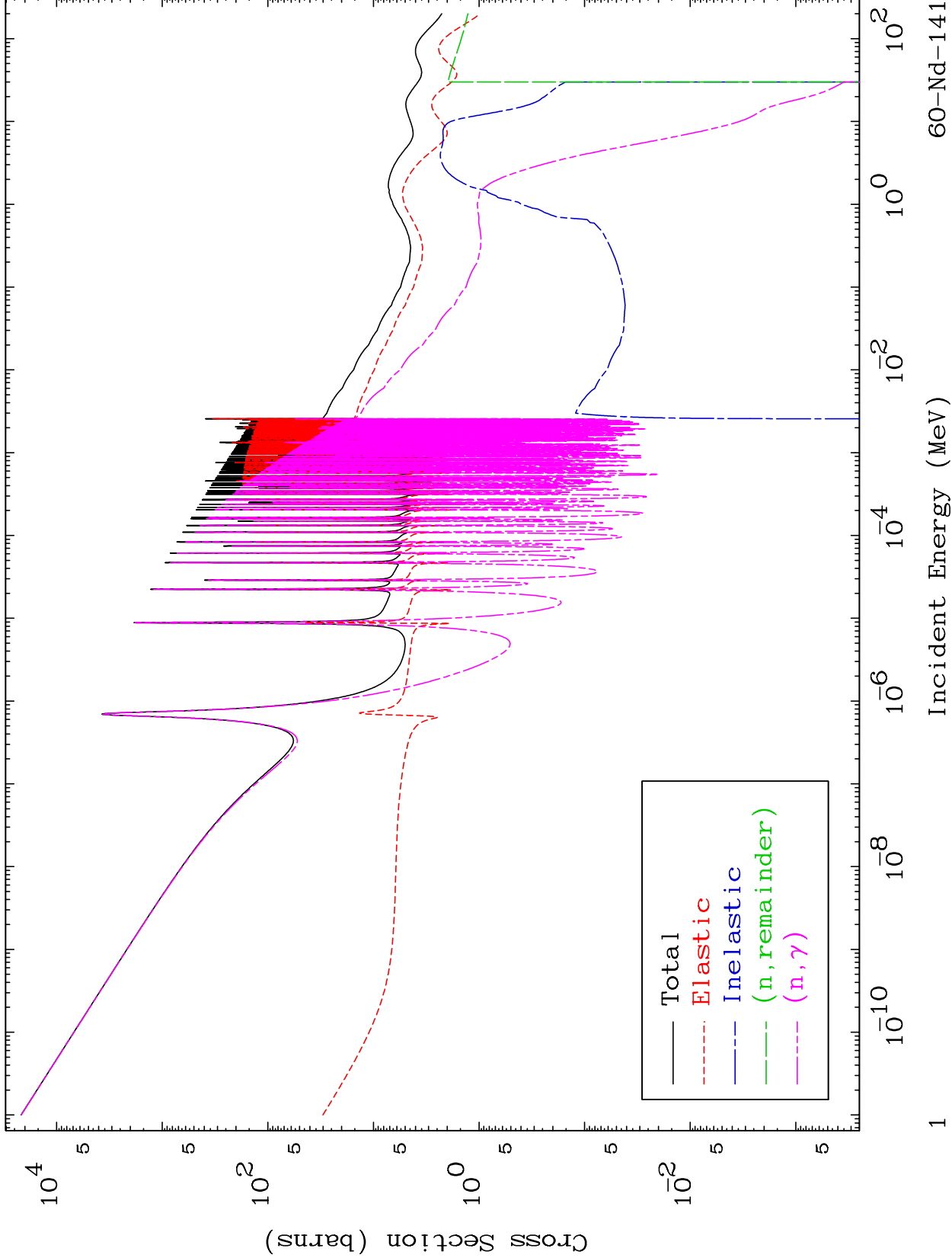
Press Mouse Button to Start

MAT 6023

Major

293 Kelvin Cross Sections

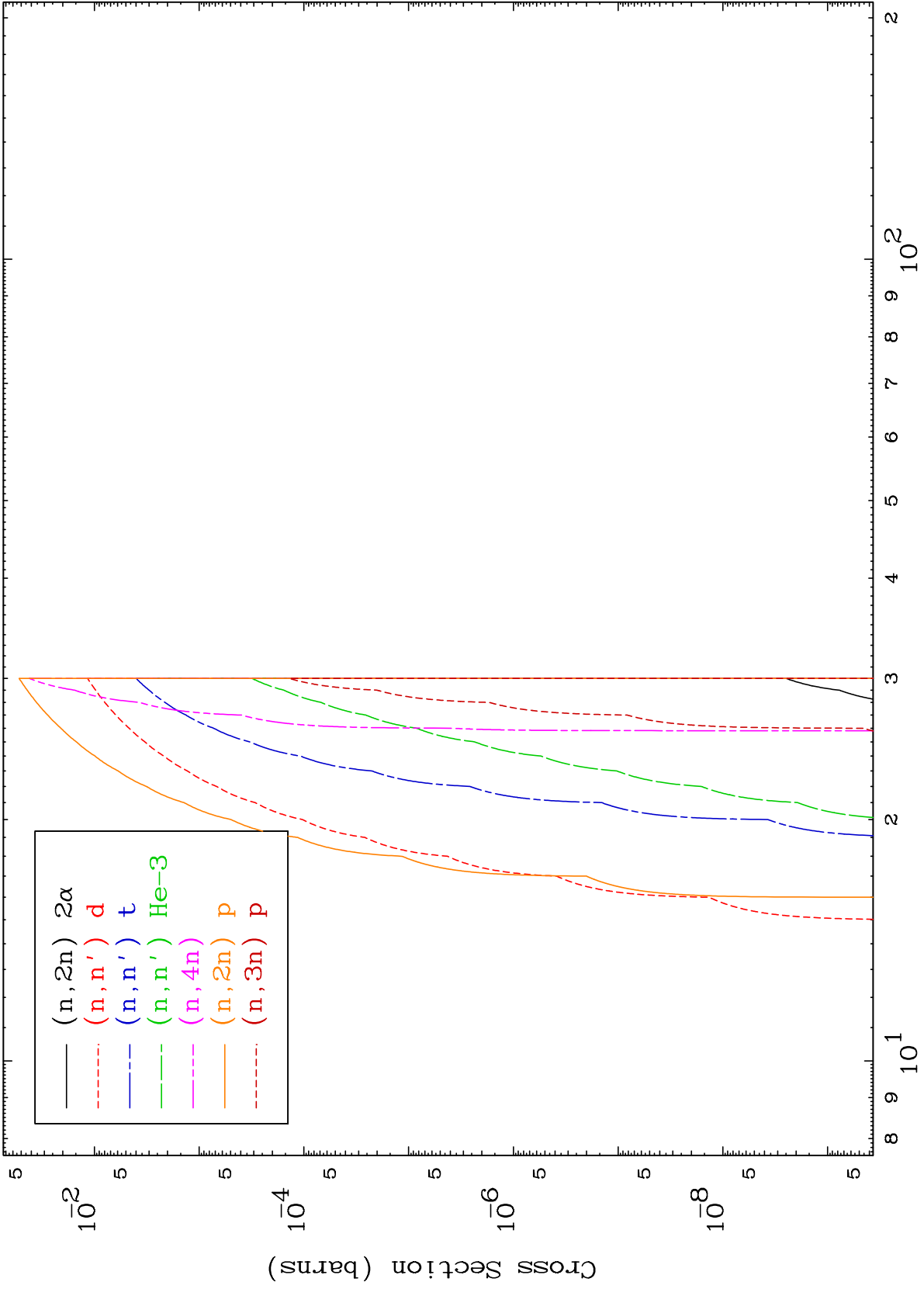
60-Nd-141



MAT 6023

Neutron Production
293 Kelvin Cross Sections

60-Nd-141



3

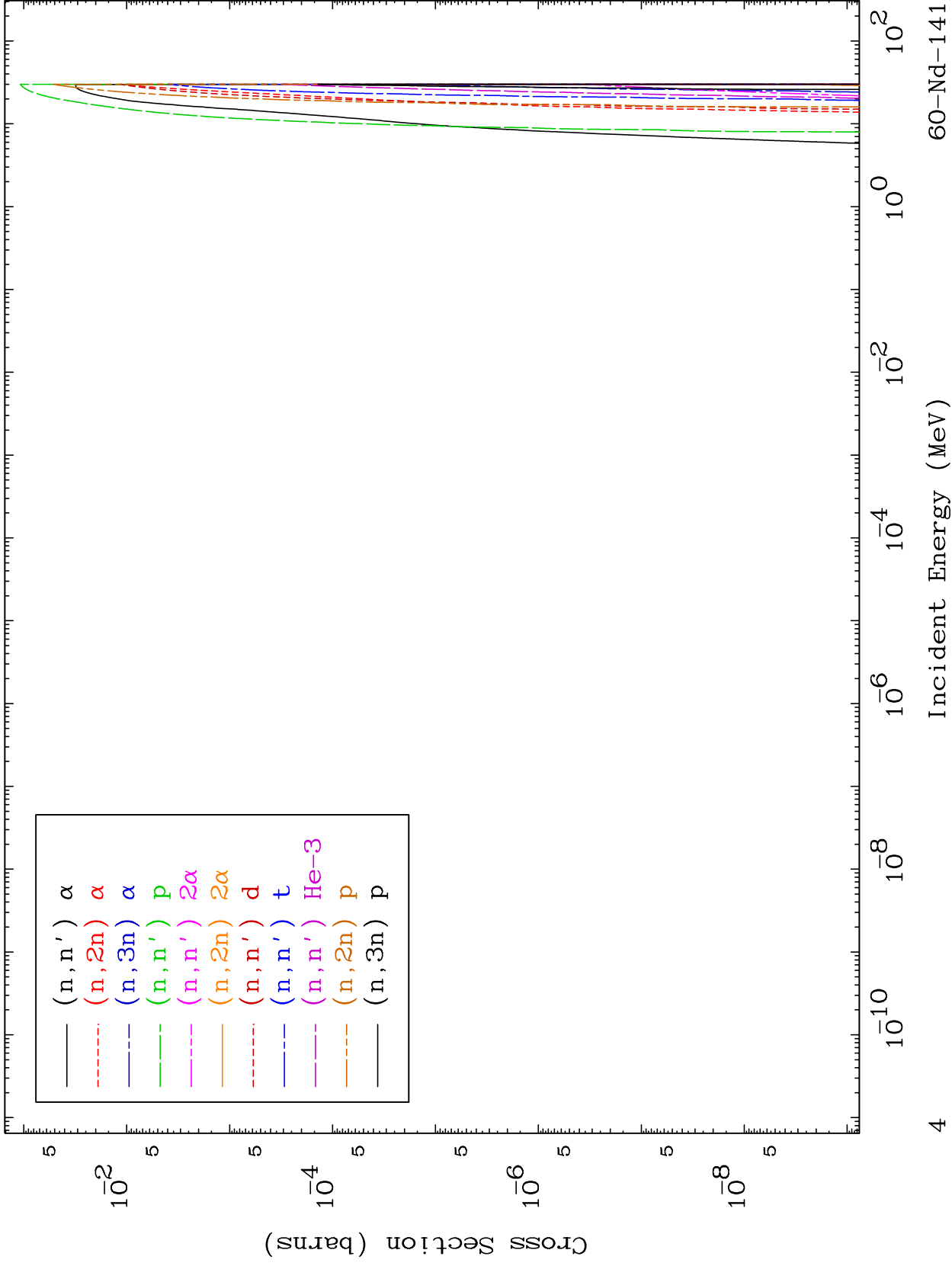
Incident Energy (MeV)

60-Nd-141

MAT 6023

Charged Particle
293 Kelvin Cross Sections

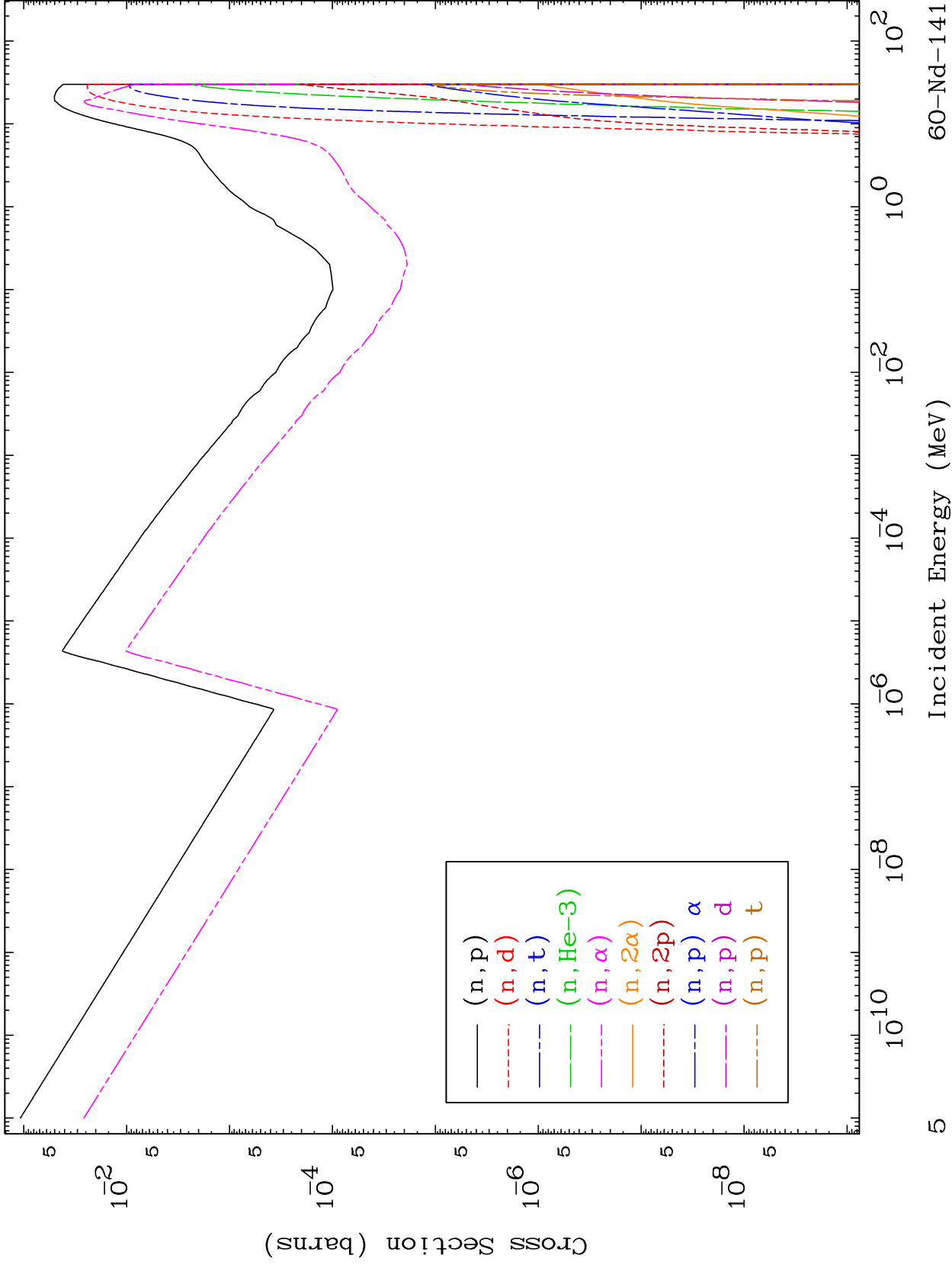
60-Nd-141

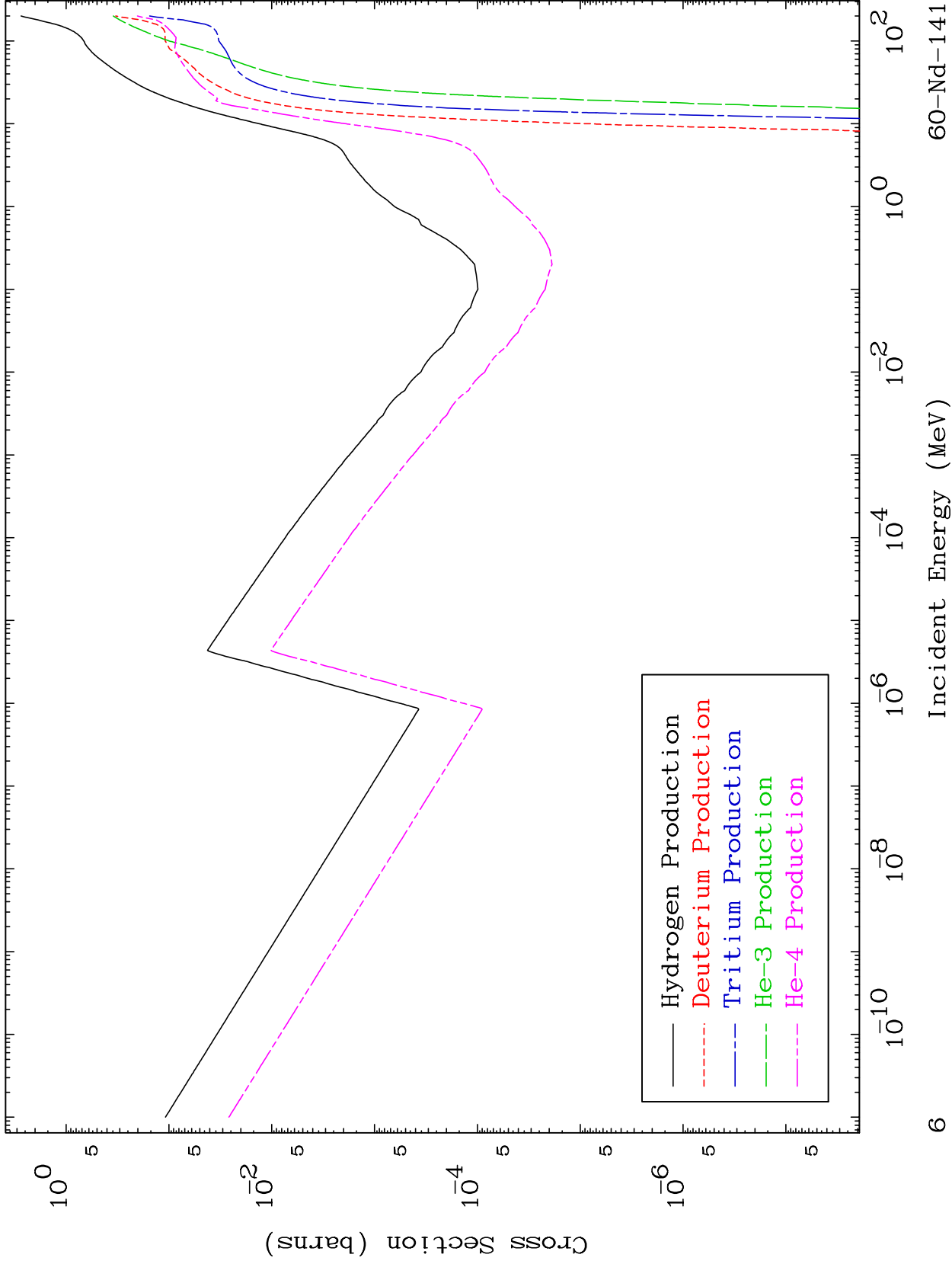


MAT 6023

Charged Particle
293 Kelvin Cross Sections

60-Nd-141

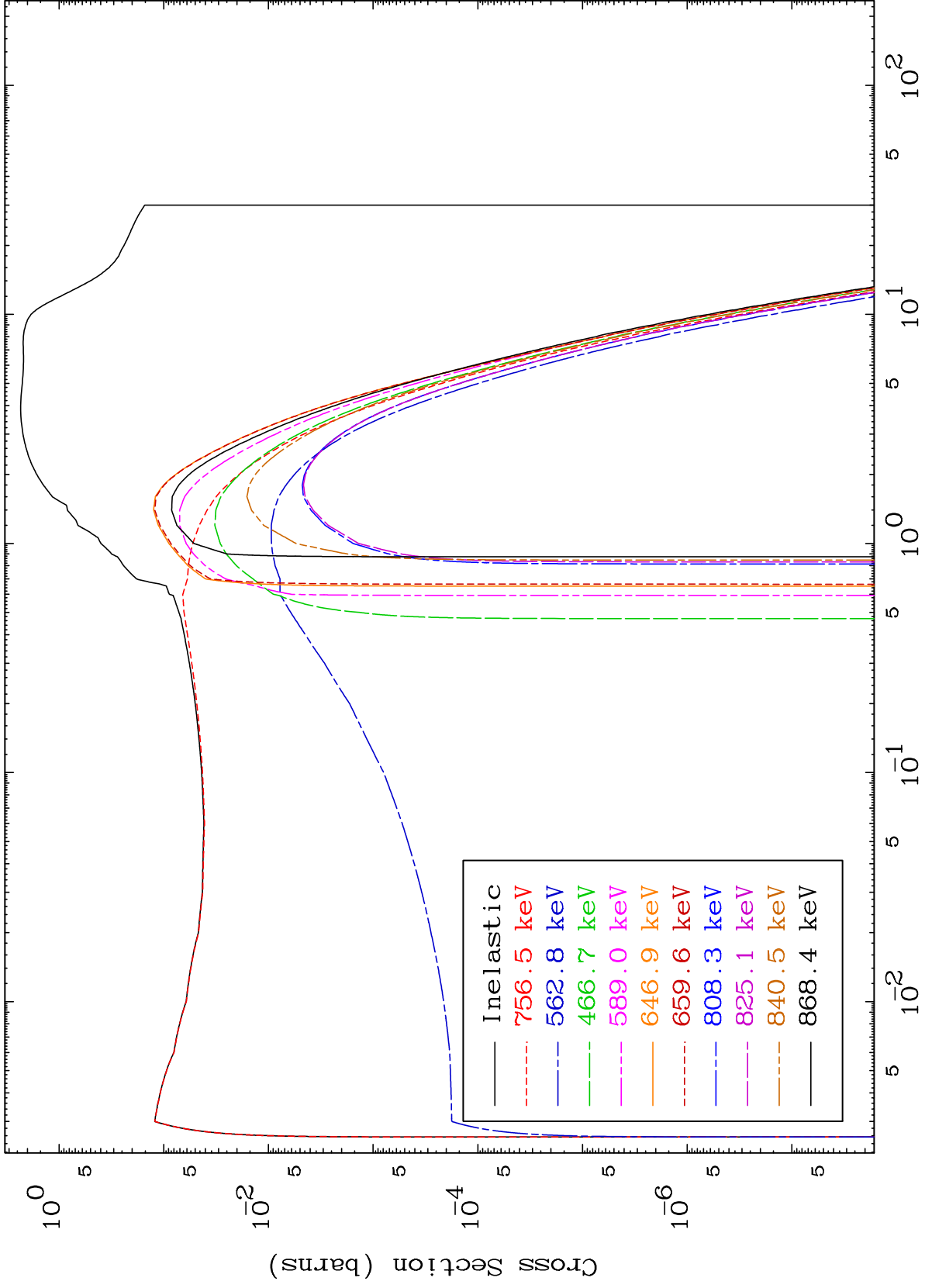




MAT 6023

(n,n') Level
293 Kelvin Cross Sections

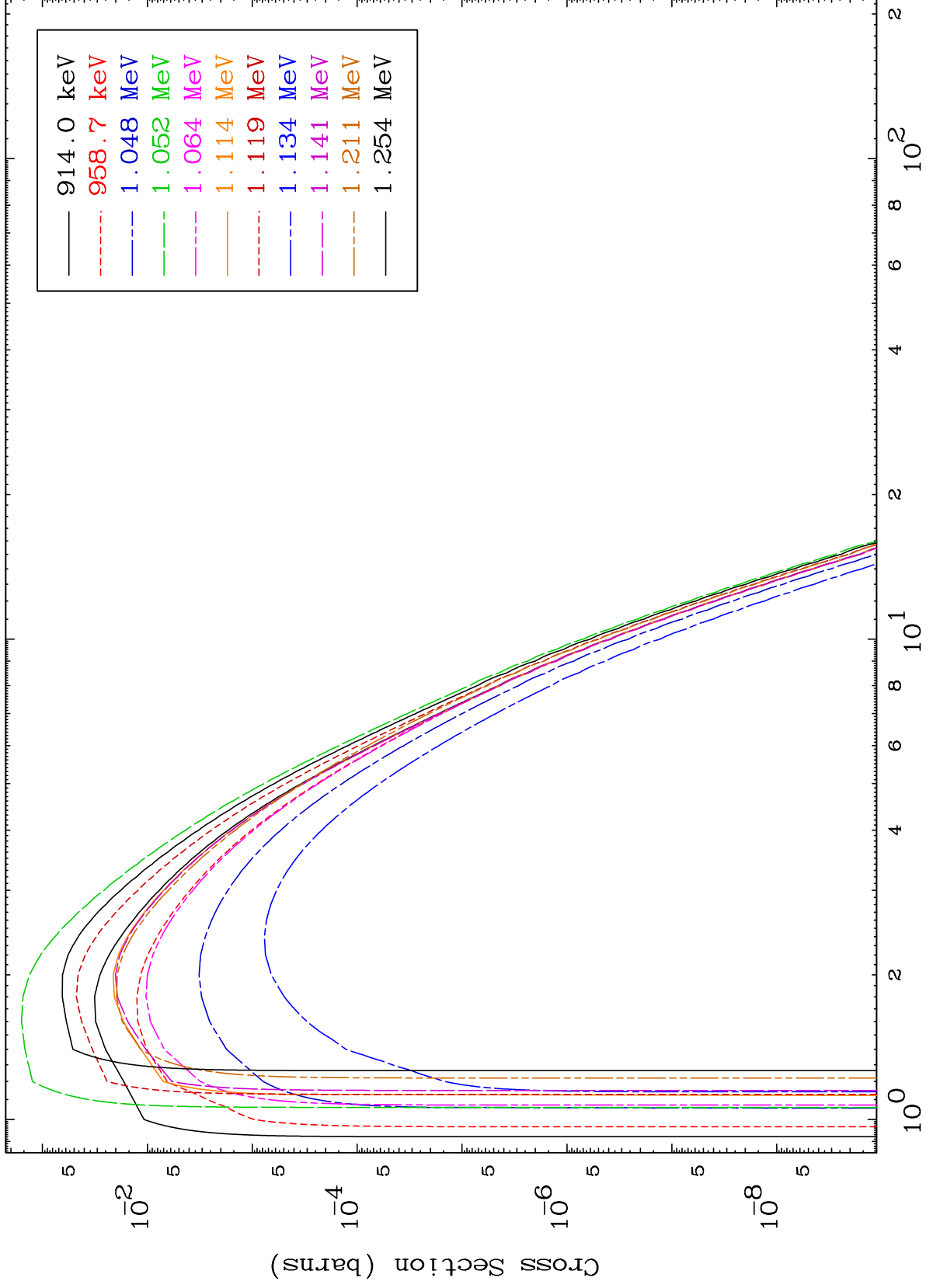
60-Nd-141



MAT 6023

(n,n') Level
293 Kelvin Cross Sections

60-Nd-141



Incident Energy (MeV)

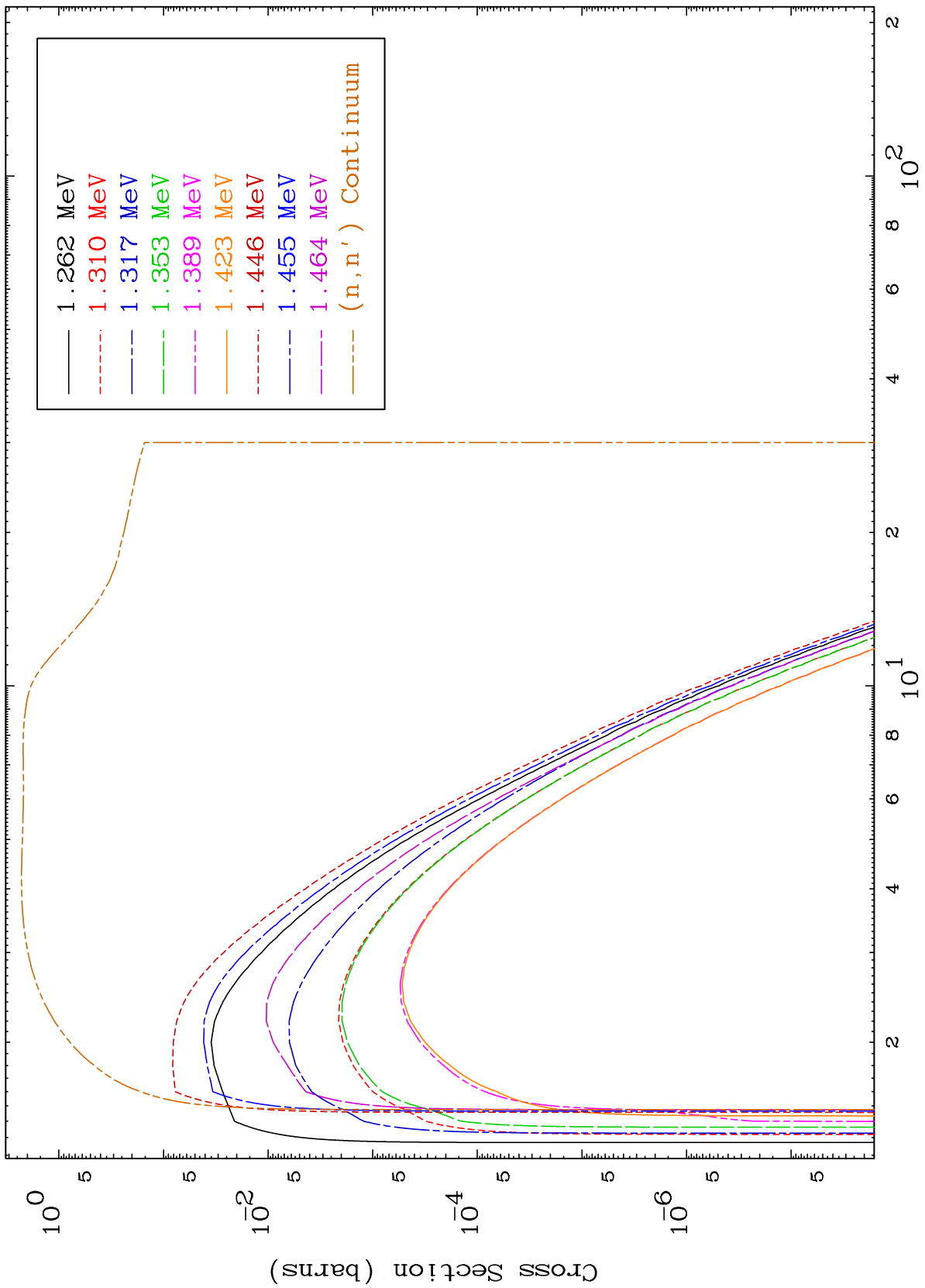
60-Nd-141

MAT 6023

(n,n') Level

60-Nd-141

293 Kelvin Cross Sections



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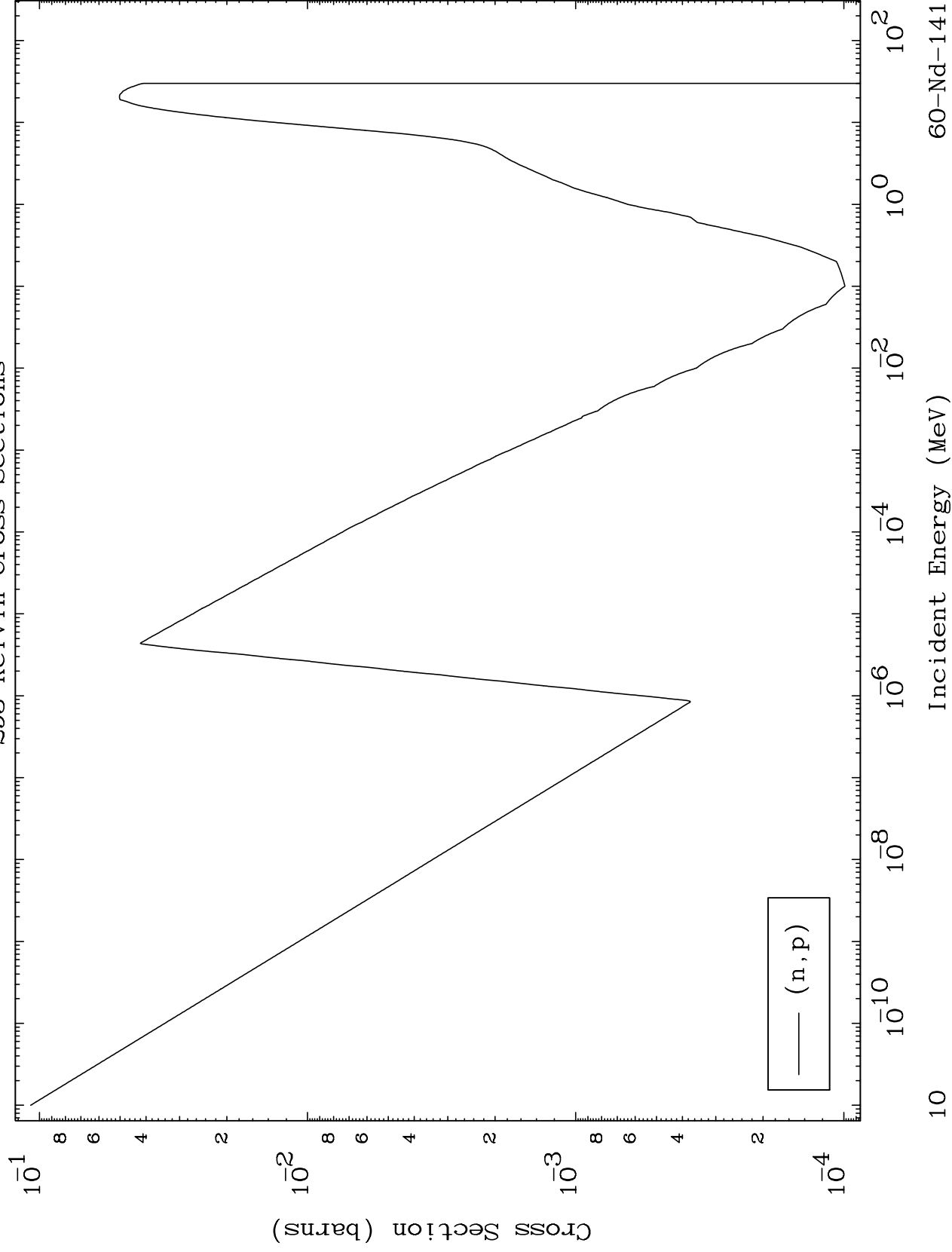
Incident Energy (MeV)

60-Nd-141

MAT 6023

(n,p) Levels
293 Kelvin Cross Sections

60-Nd-141



60-Nd-141

Incident Energy (MeV)

10

MAT 6023

(n,d) Levels
293 Kelvin Cross Sections

60-Nd-141



11

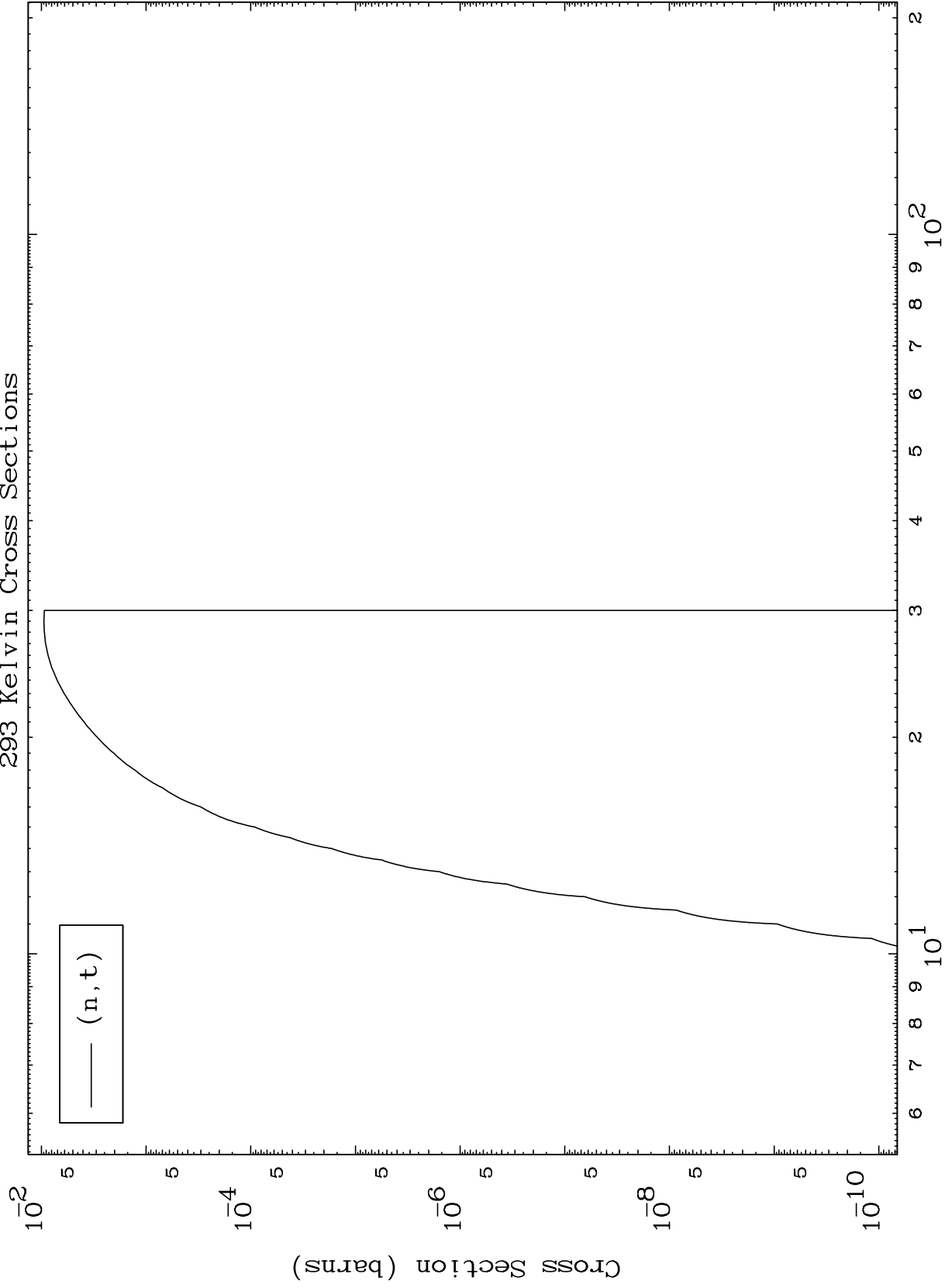
Incident Energy (MeV)

60-Nd-141

MAT 6023

(n, t) Levels
293 Kelvin Cross Sections

60-Nd-141



12

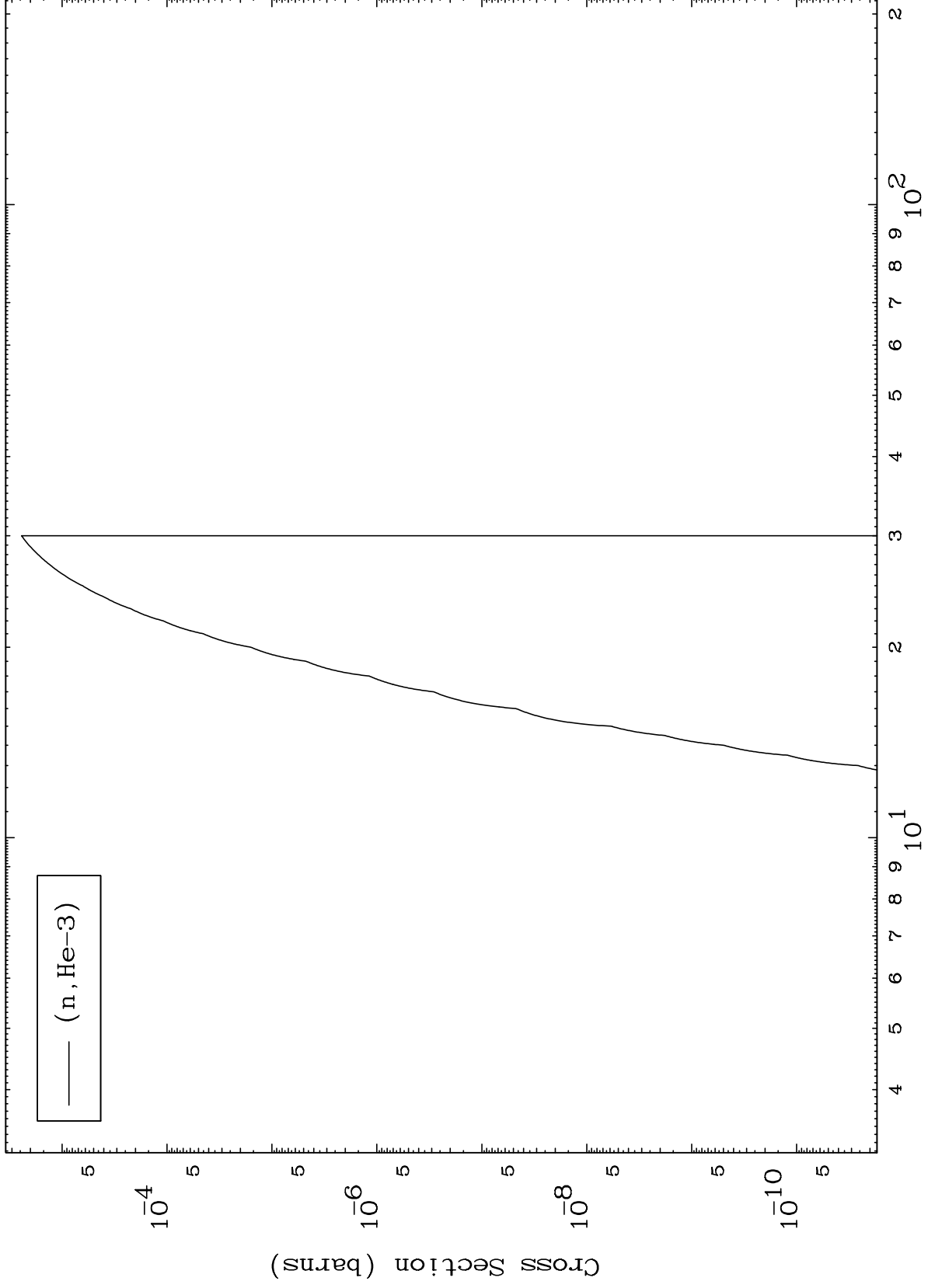
Incident Energy (MeV)

60-Nd-141

MAT 6023

(n,He3) Levels
293 Kelvin Cross Sections

60-Nd-141



13

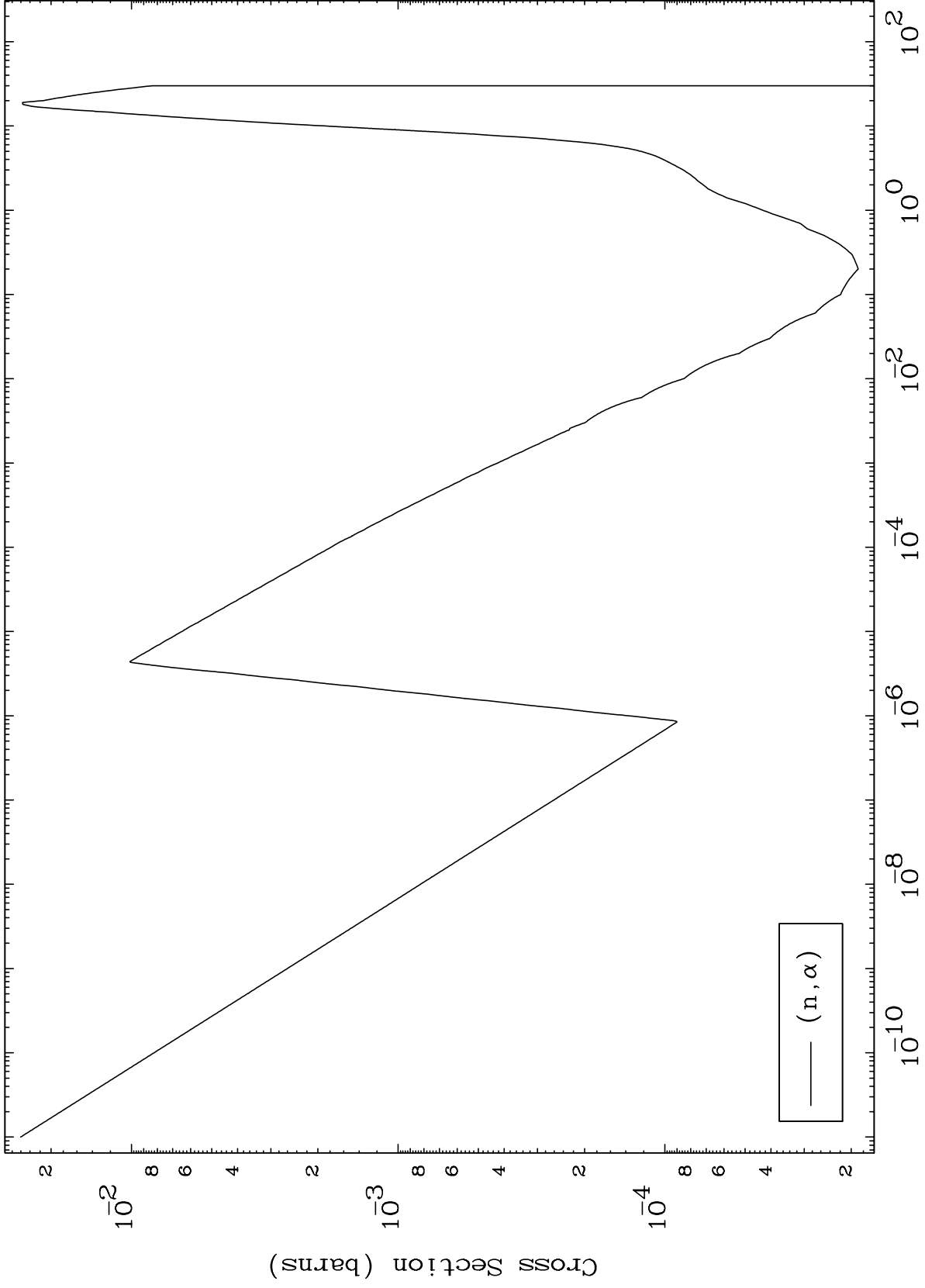
Incident Energy (MeV)

60-Nd-141

MAT 6023

(n, α) Levels
293 Kelvin Cross Sections

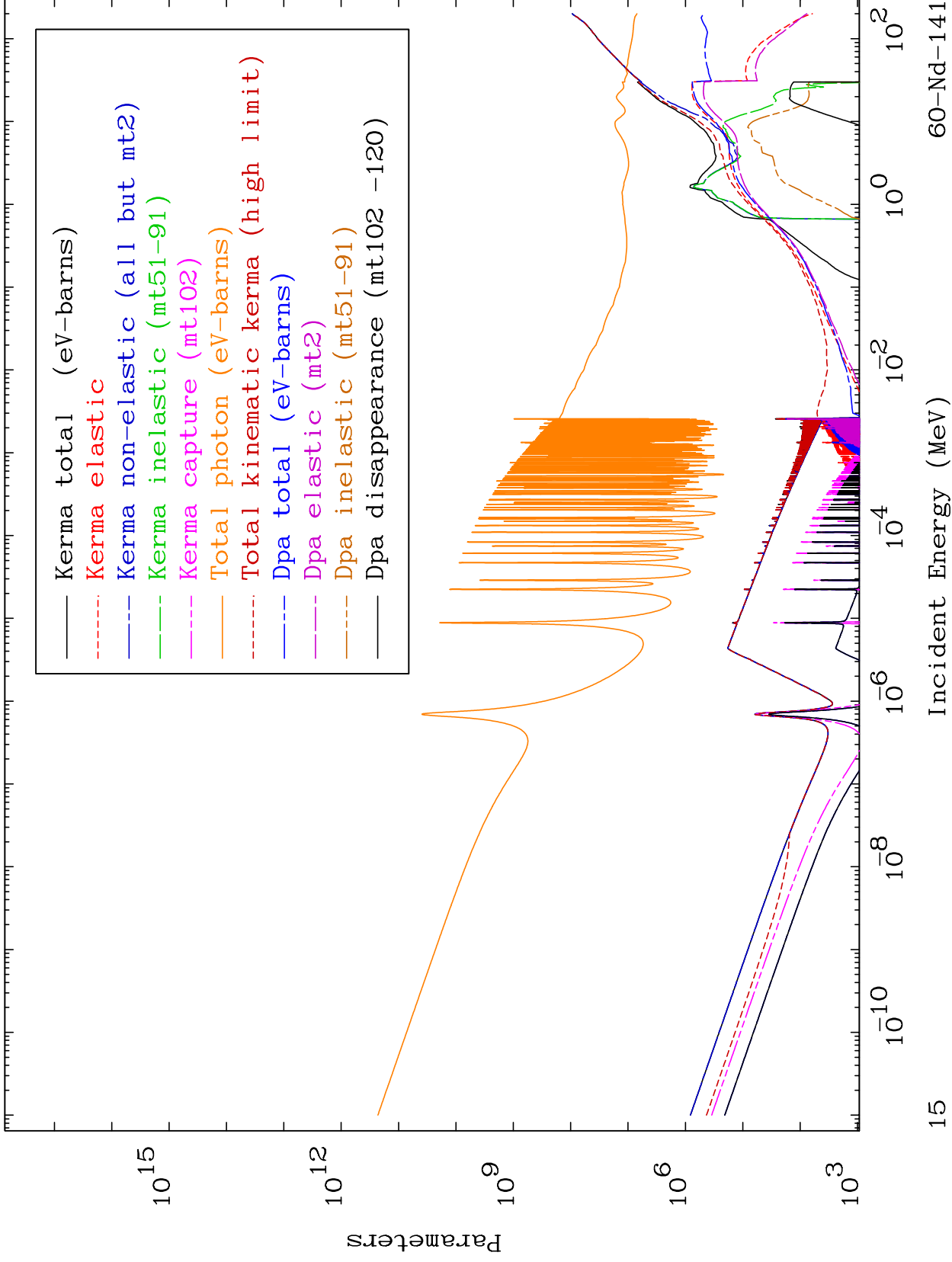
60-Nd-141



14

Incident Energy (MeV)

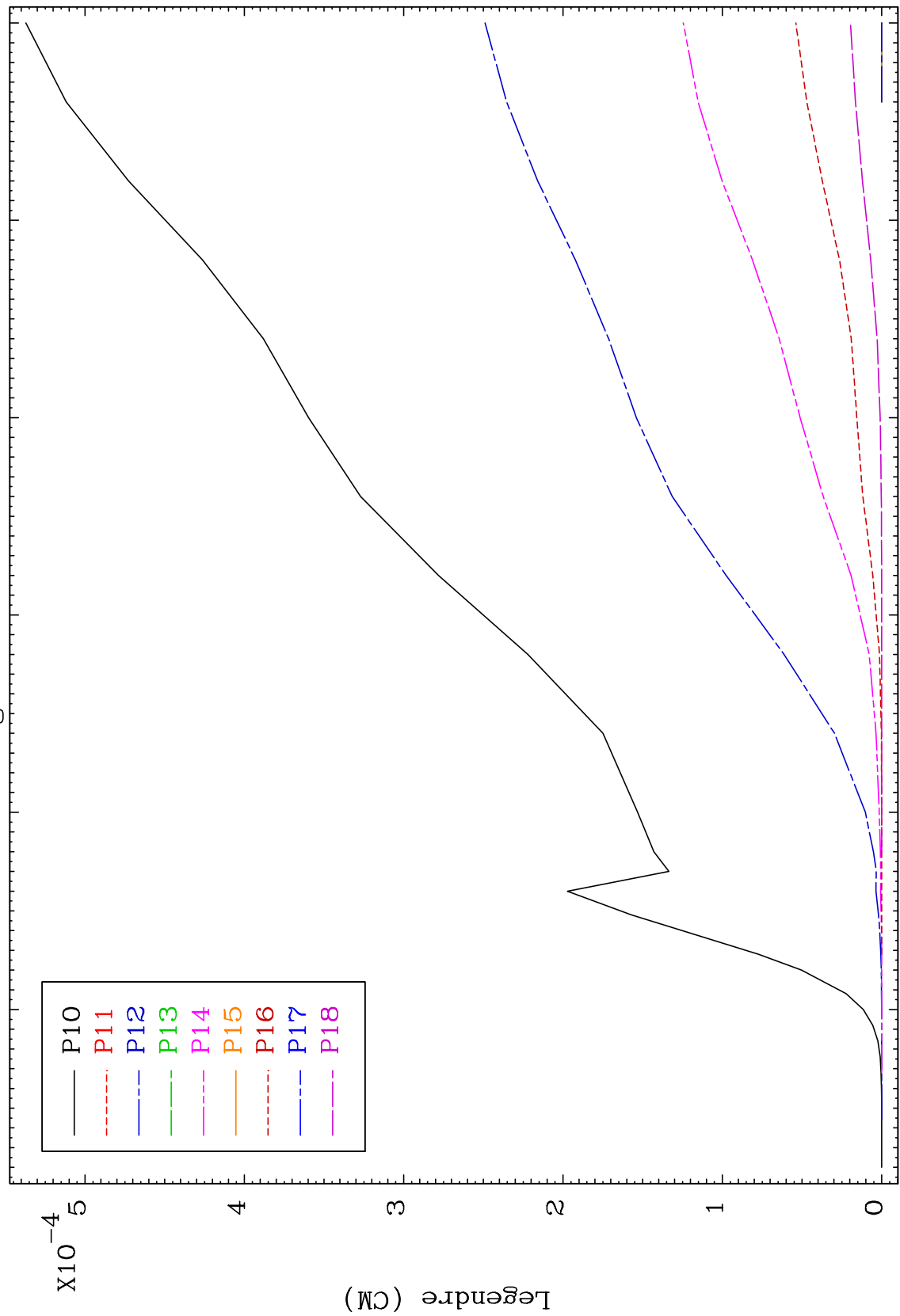
60-Nd-141



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Elastic Legendre Coefficients

⁶⁰Nd-141



17

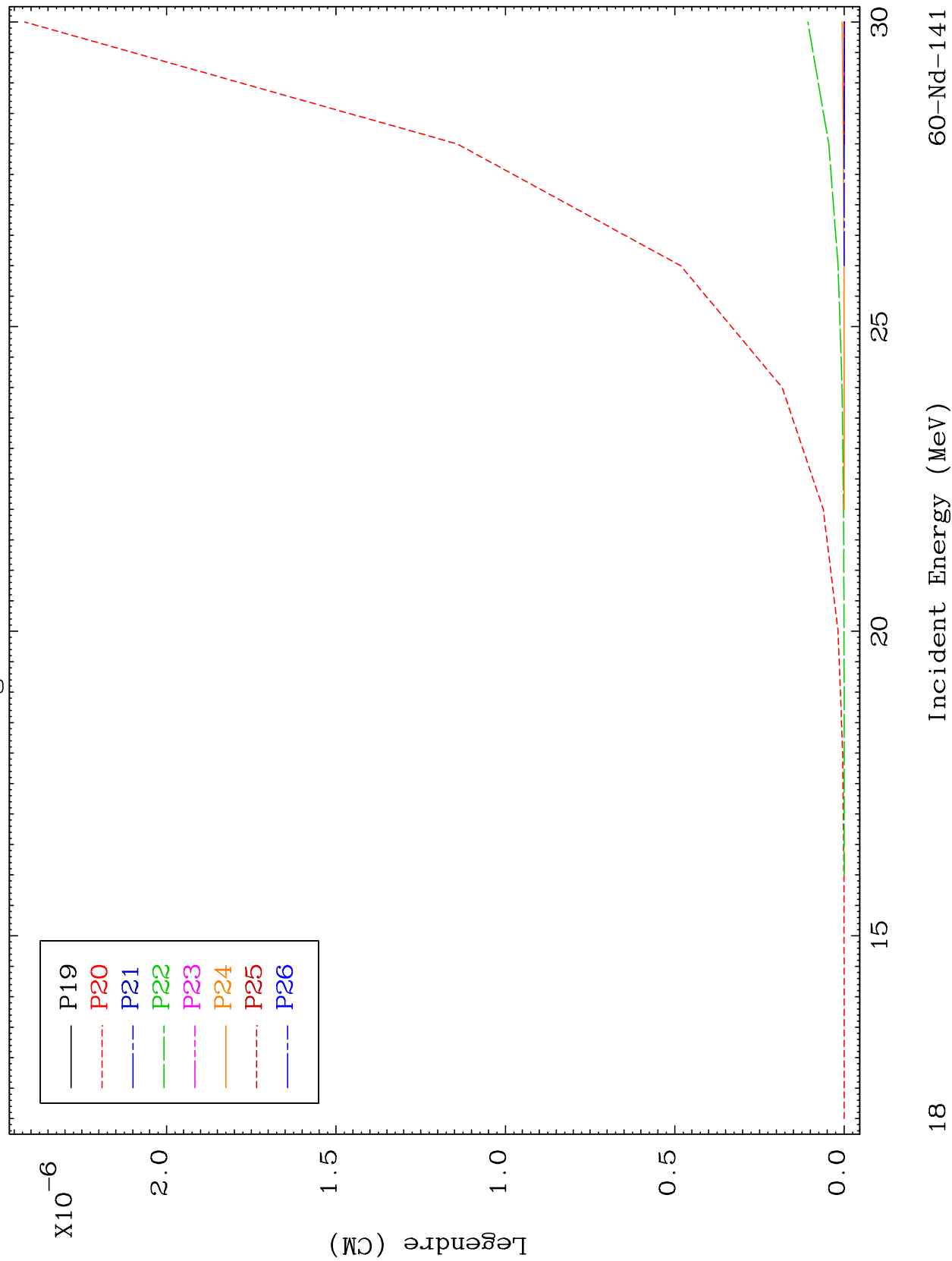
Incident Energy (MeV)

⁶⁰Nd-141

MAT 6023

Elastic
Legendre Coefficients

60-Nd-141



18

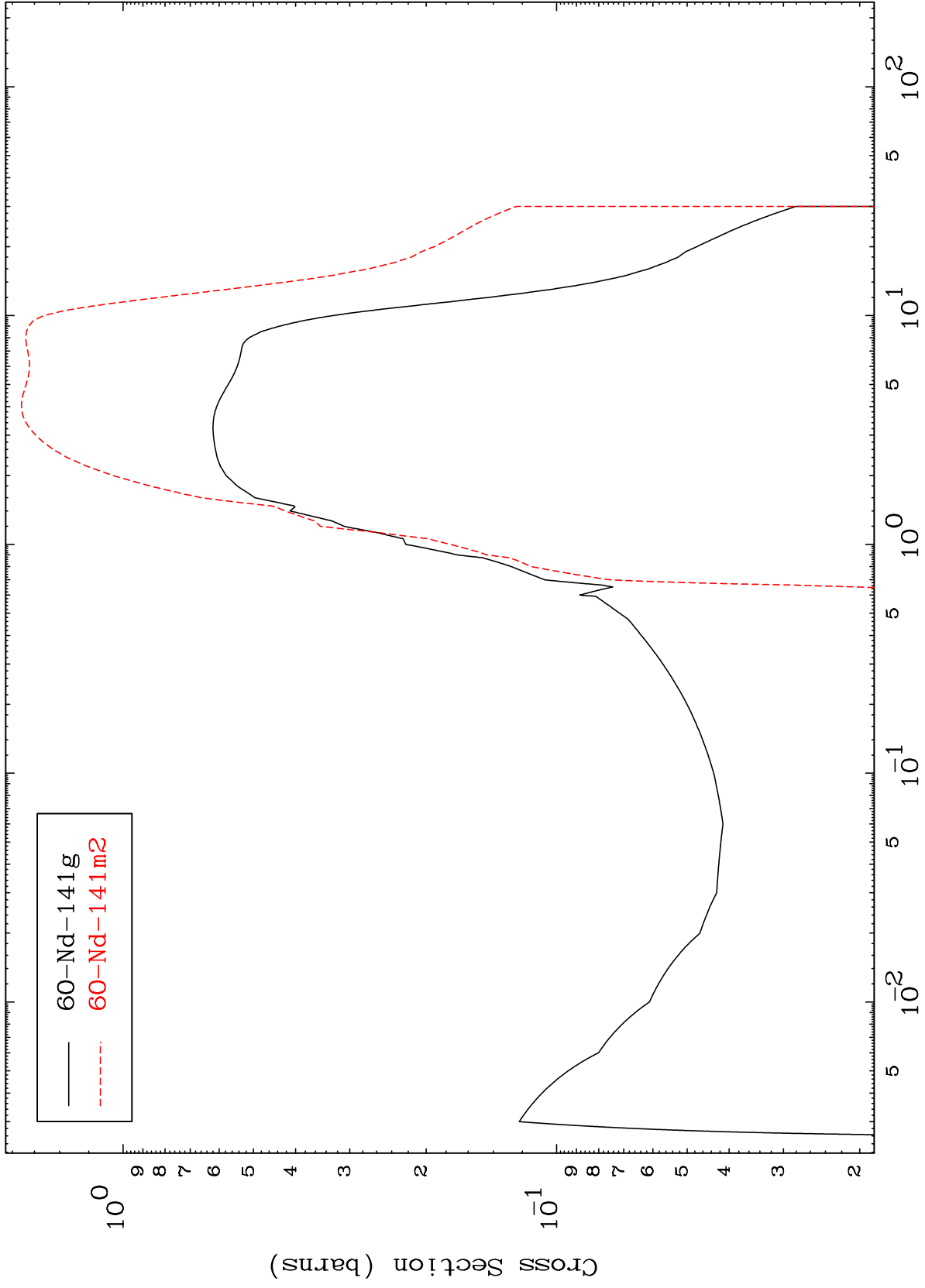
Incident Energy (MeV)

60-Nd-141

MAT 6023

60-Nd-141

Inelastic
Radionuclide Production Cross Section

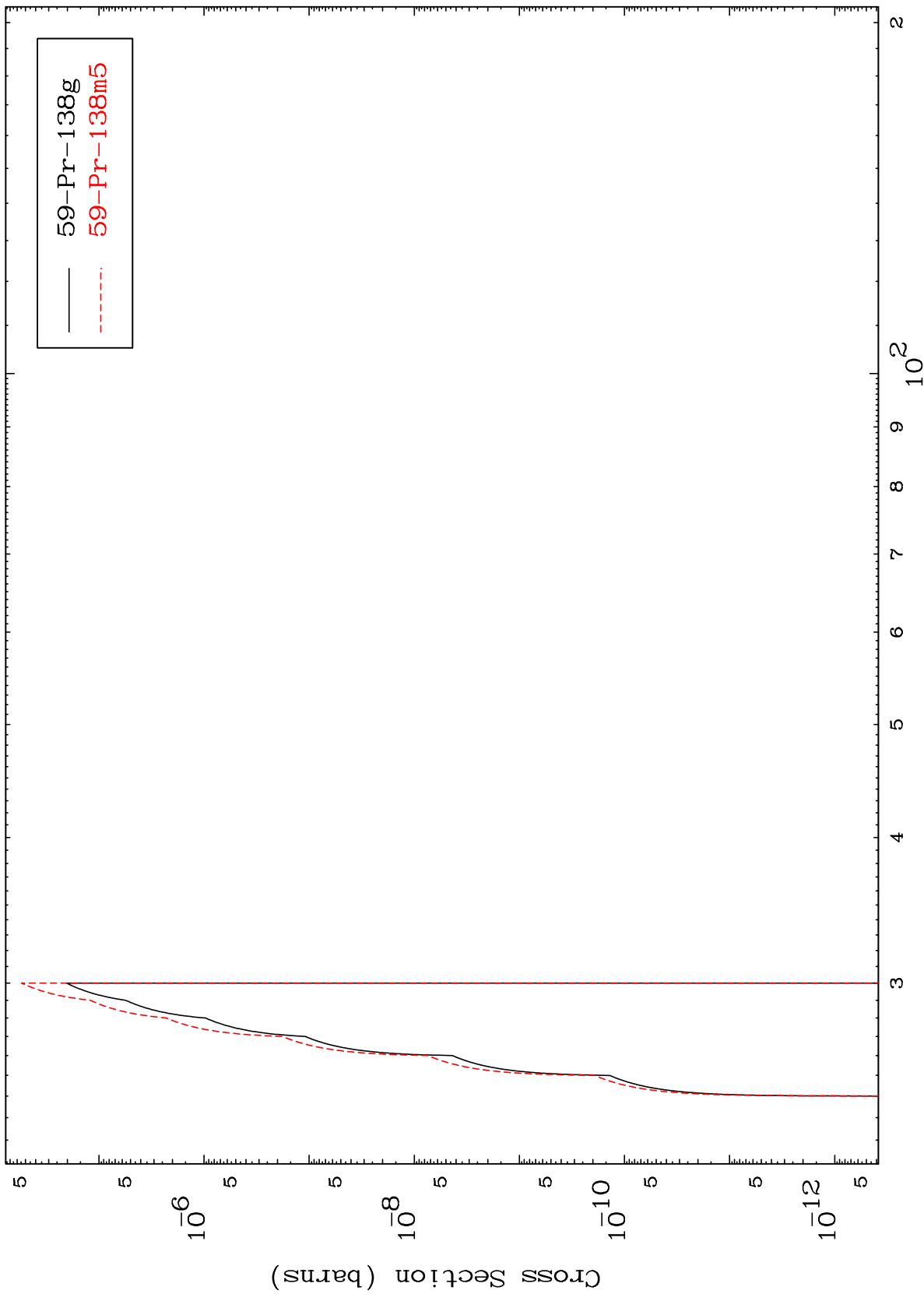


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(n,2n) d

60-Nd-141

Radionuclide Production Cross Section



20

Incident Energy (MeV)

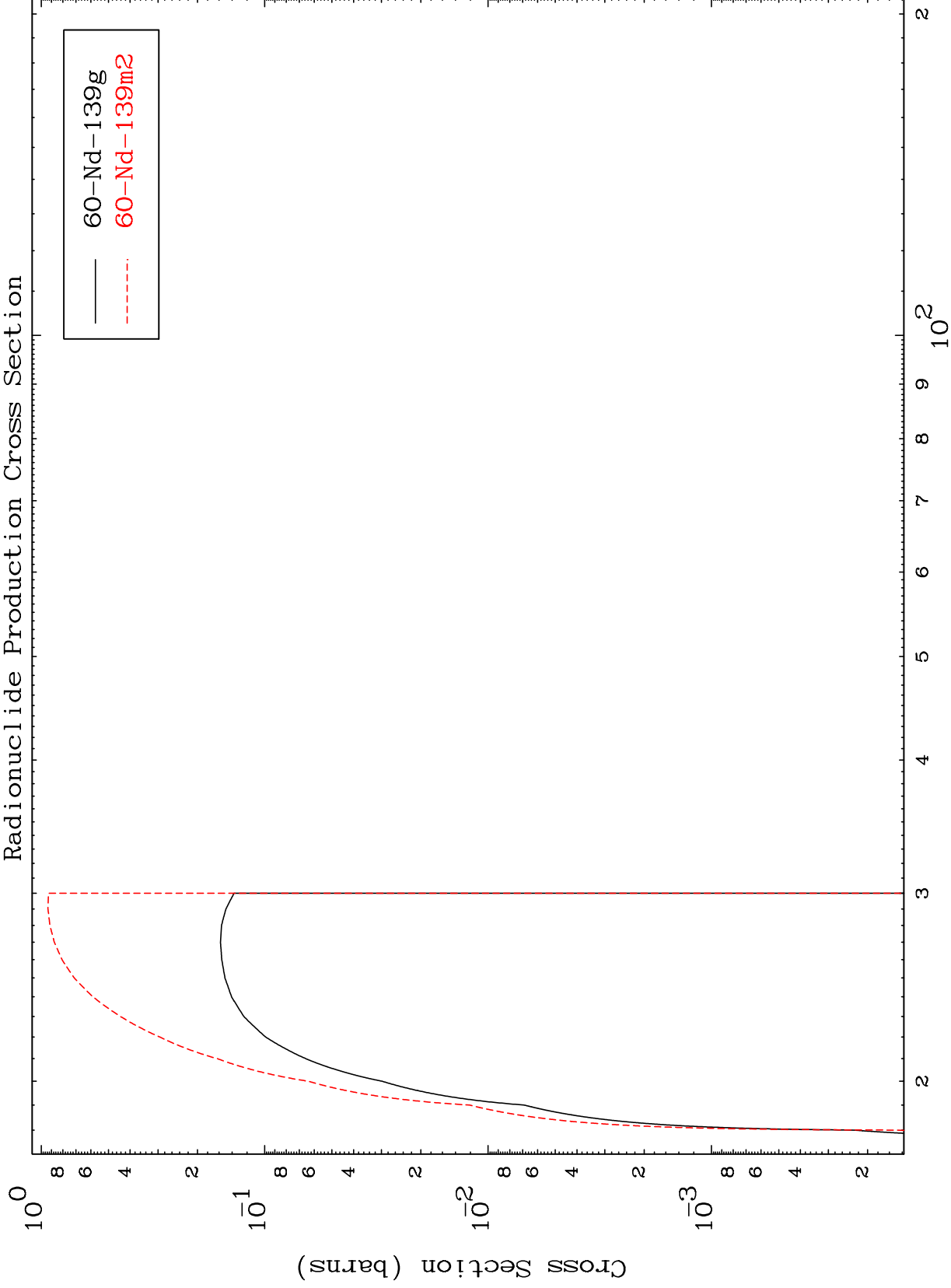
60-Nd-141

MAT 6023

(n,3n)

60-Nd-141

Radionuclide Production Cross Section



21

Incident Energy (MeV)

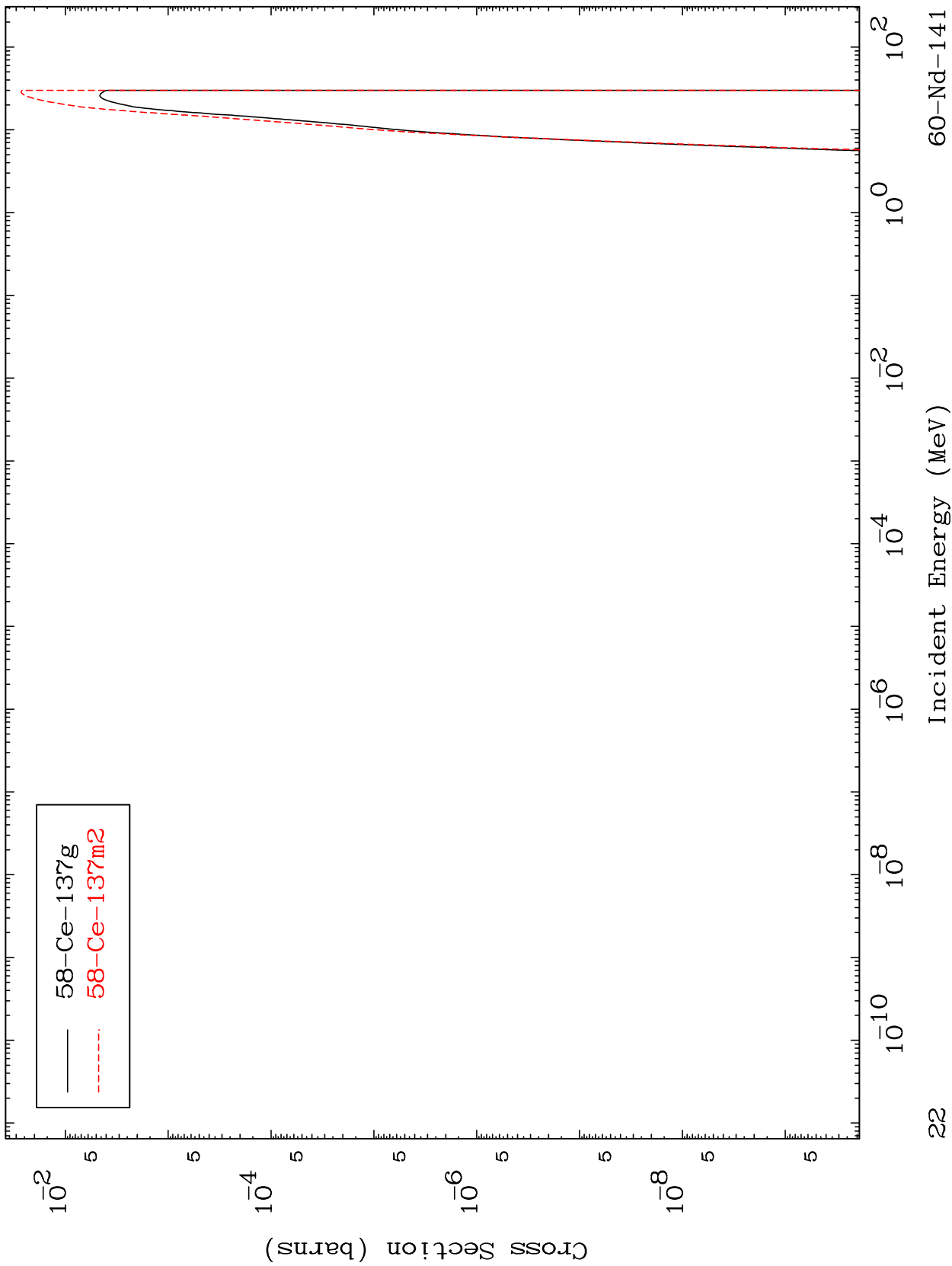
60-Nd-141

MAT 6023

(n, n') α

60-Nd-141

Radionuclide Production Cross Section

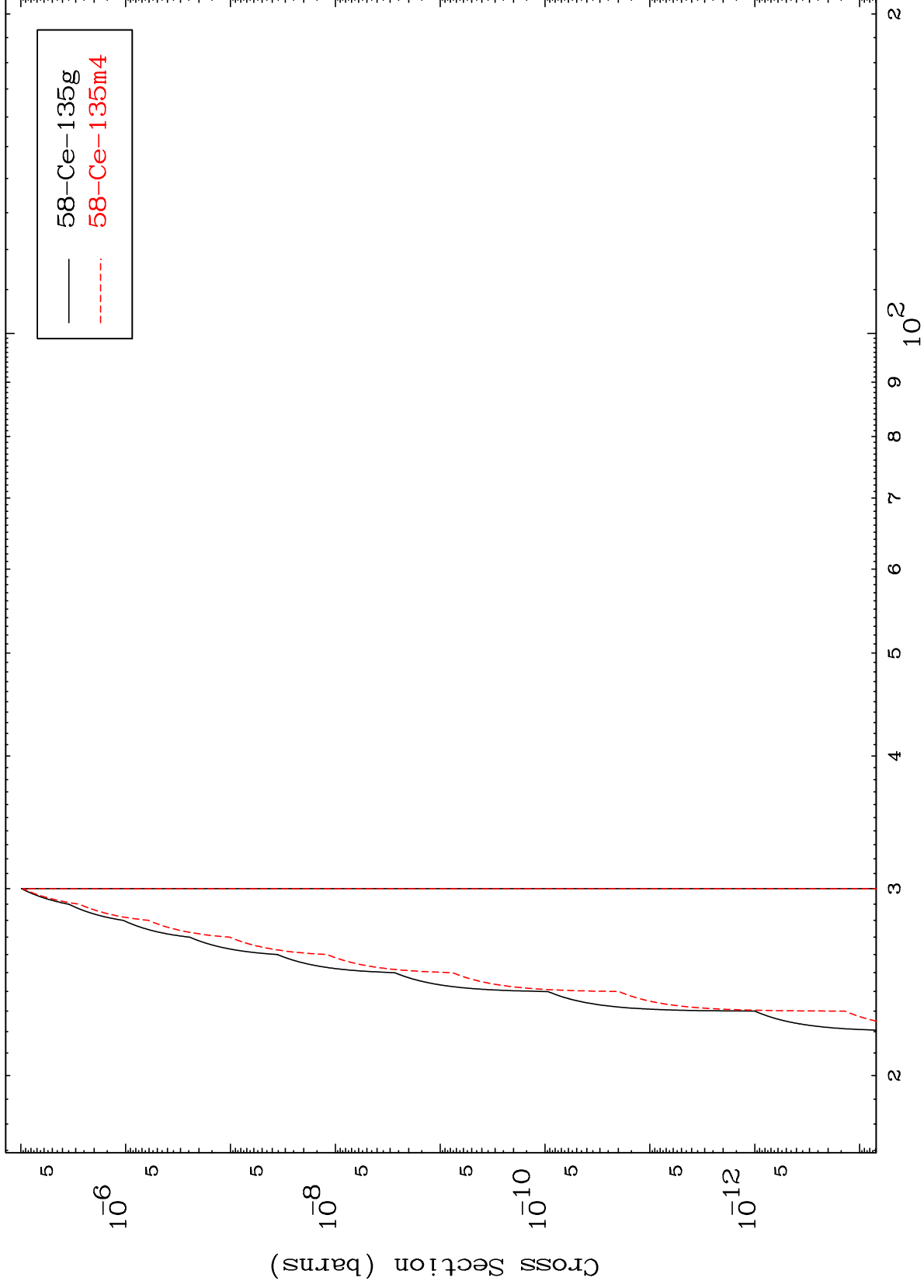


MAT 6023

$(n,3n) \alpha$

60-Nd-141

Radionuclide Production Cross Section



23

Incident Energy (MeV)

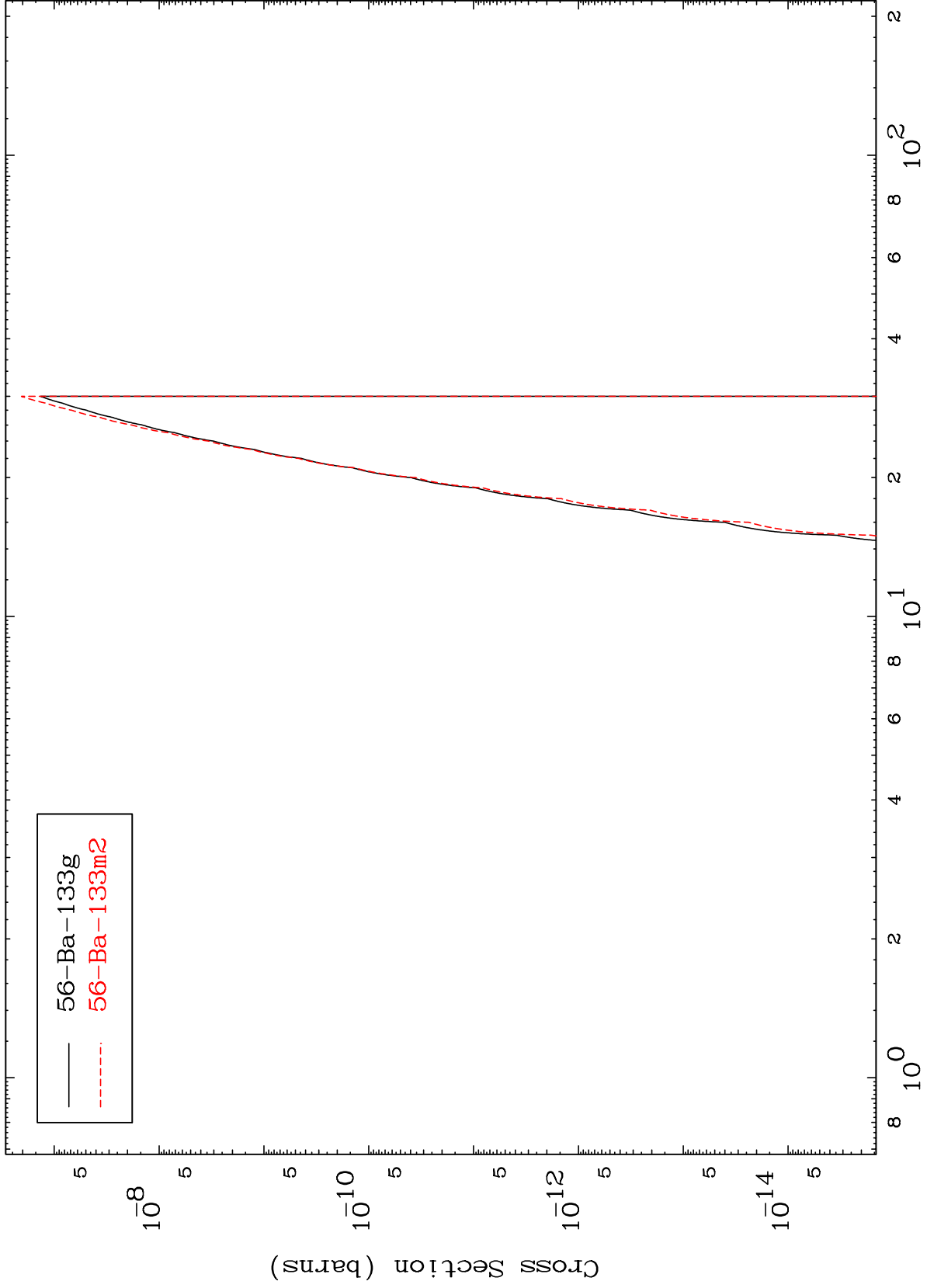
60-Nd-141

MAT 6023

(n,n') 2α

60-Nd-141

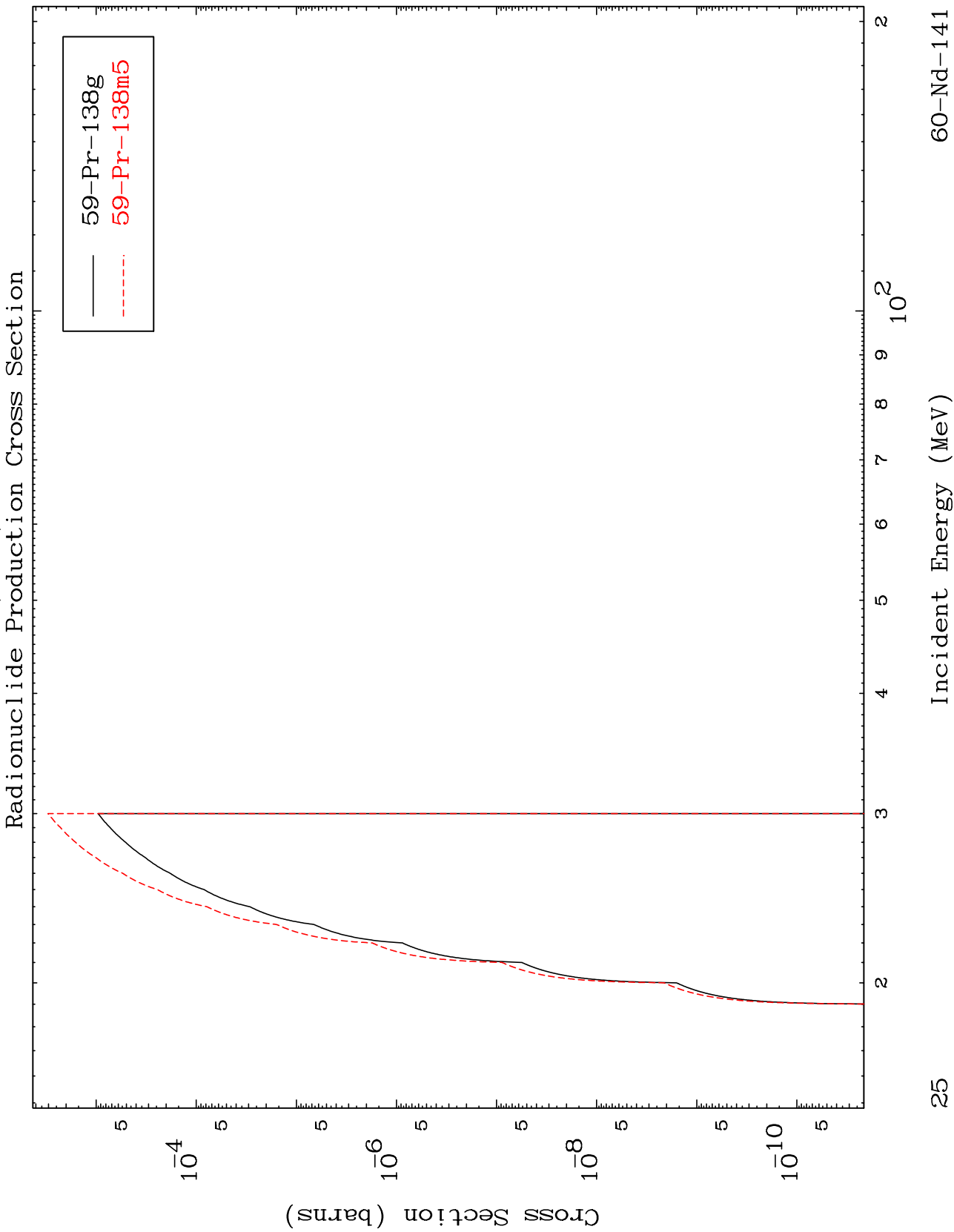
Radionuclide Production Cross Section



24

Incident Energy (MeV)

60-Nd-141

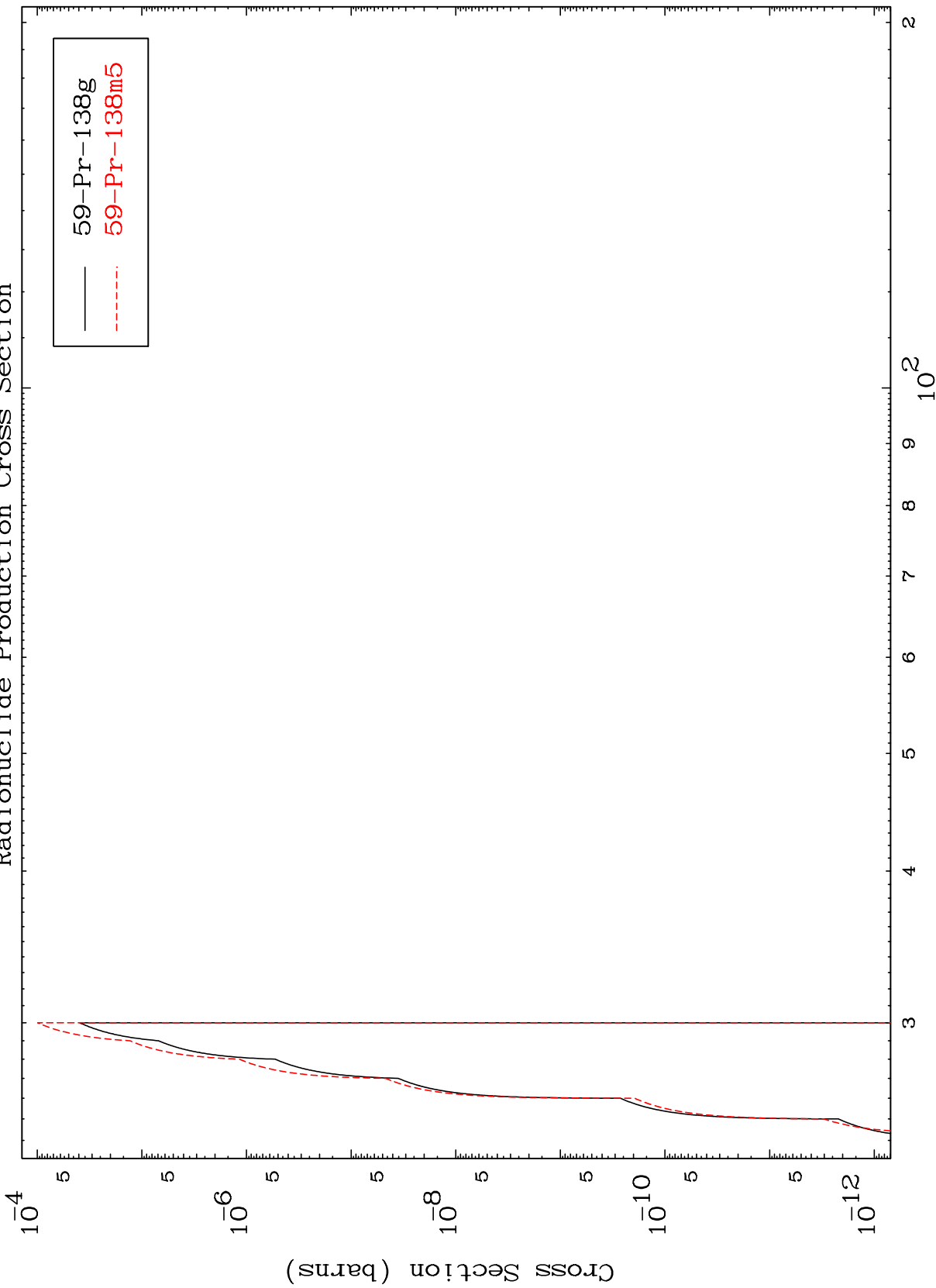


MAT 6023

(n,3n) p

60-Nd-141

Radionuclide Production Cross Section



26

Incident Energy (MeV)

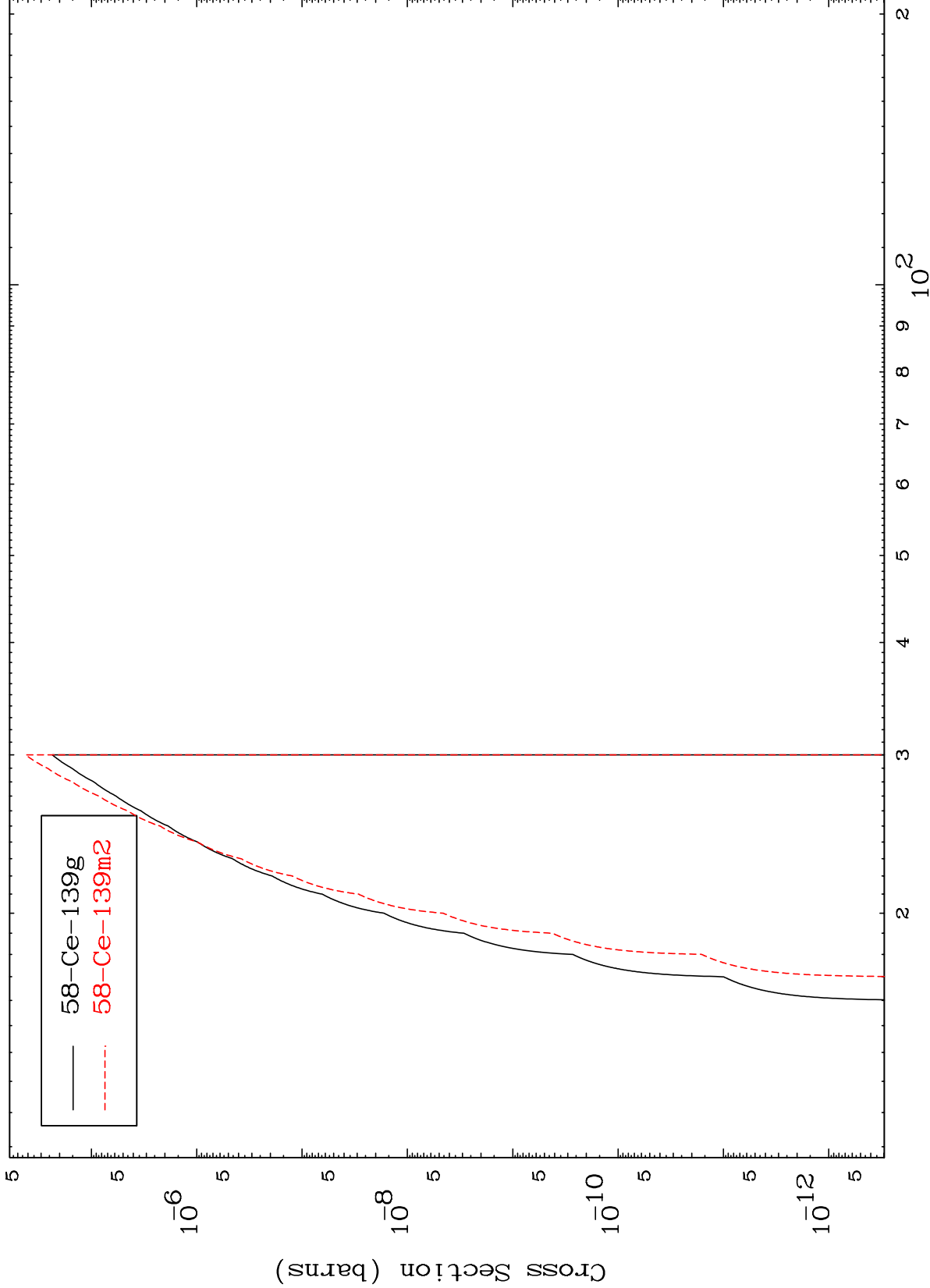
60-Nd-141

MAT 6023

(n,2n) p

60-Nd-141

Radionuclide Production Cross Section

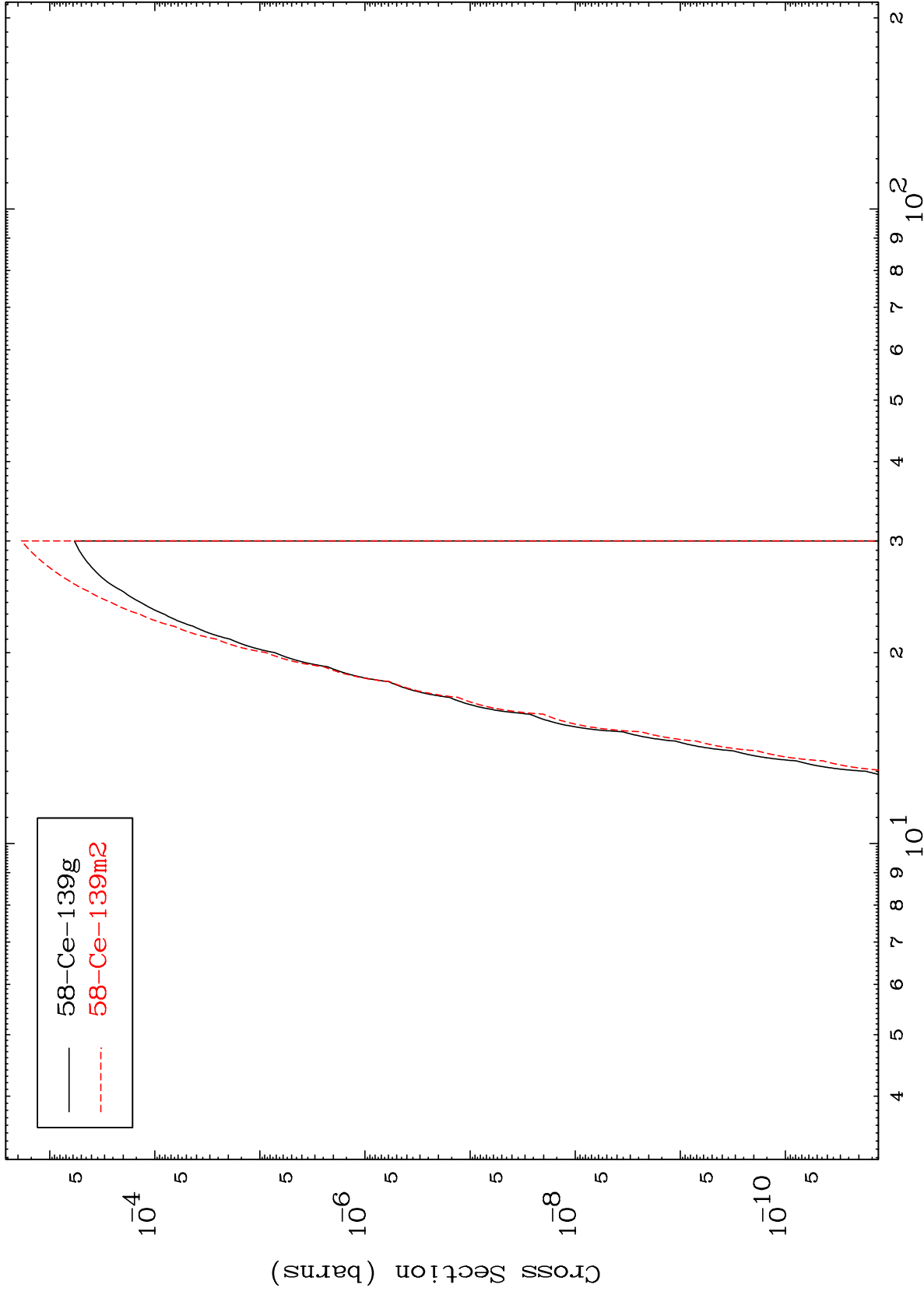


MAT 6023

(n,He-3)

60-Nd-141

Radionuclide Production Cross Section



28

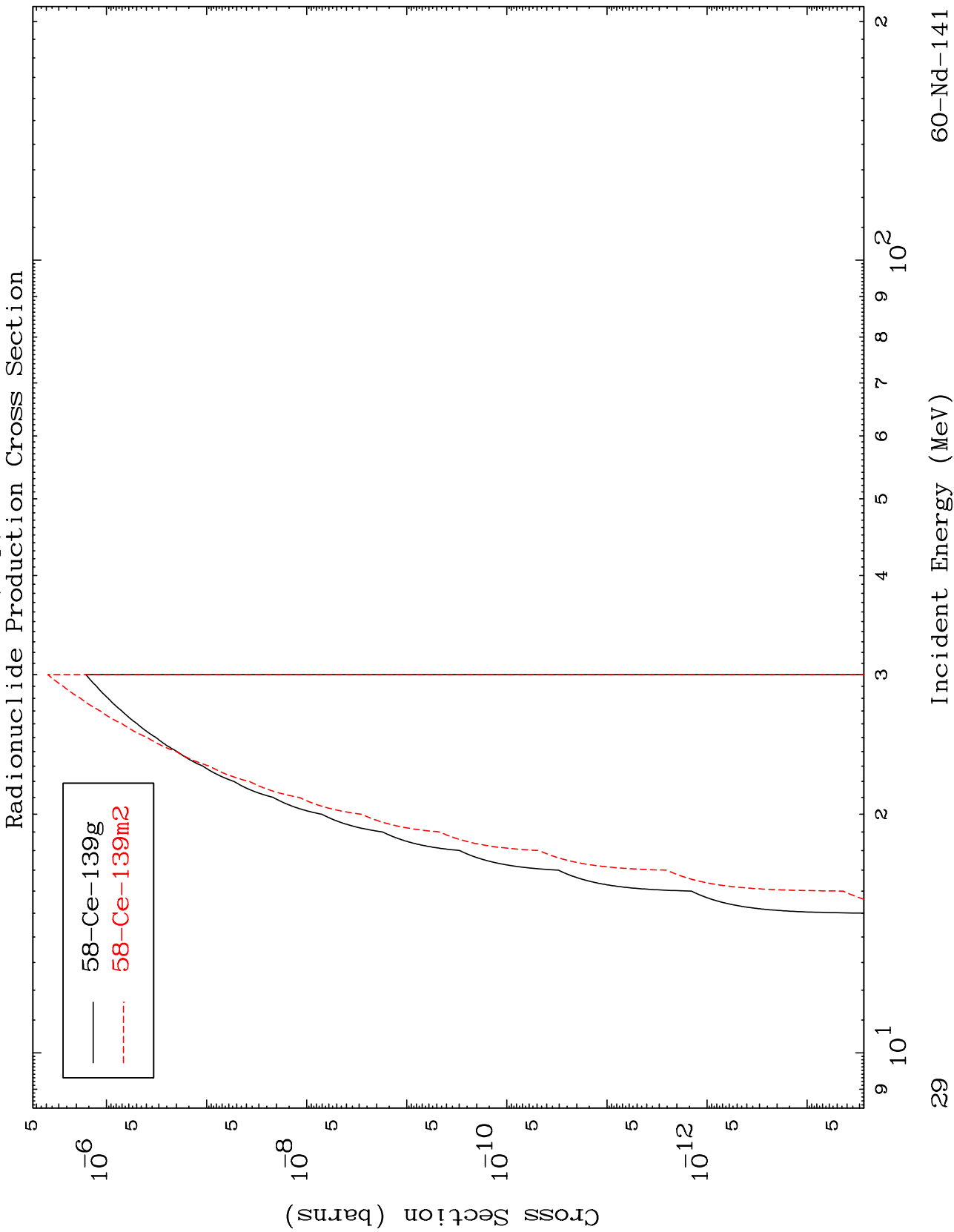
Incident Energy (MeV)

60-Nd-141

MAT 6023

(n,p) d

60-Nd-141



29

Incident Energy (MeV)

60-Nd-141