

Program EVALPLOT
(Version 2015-2)

by

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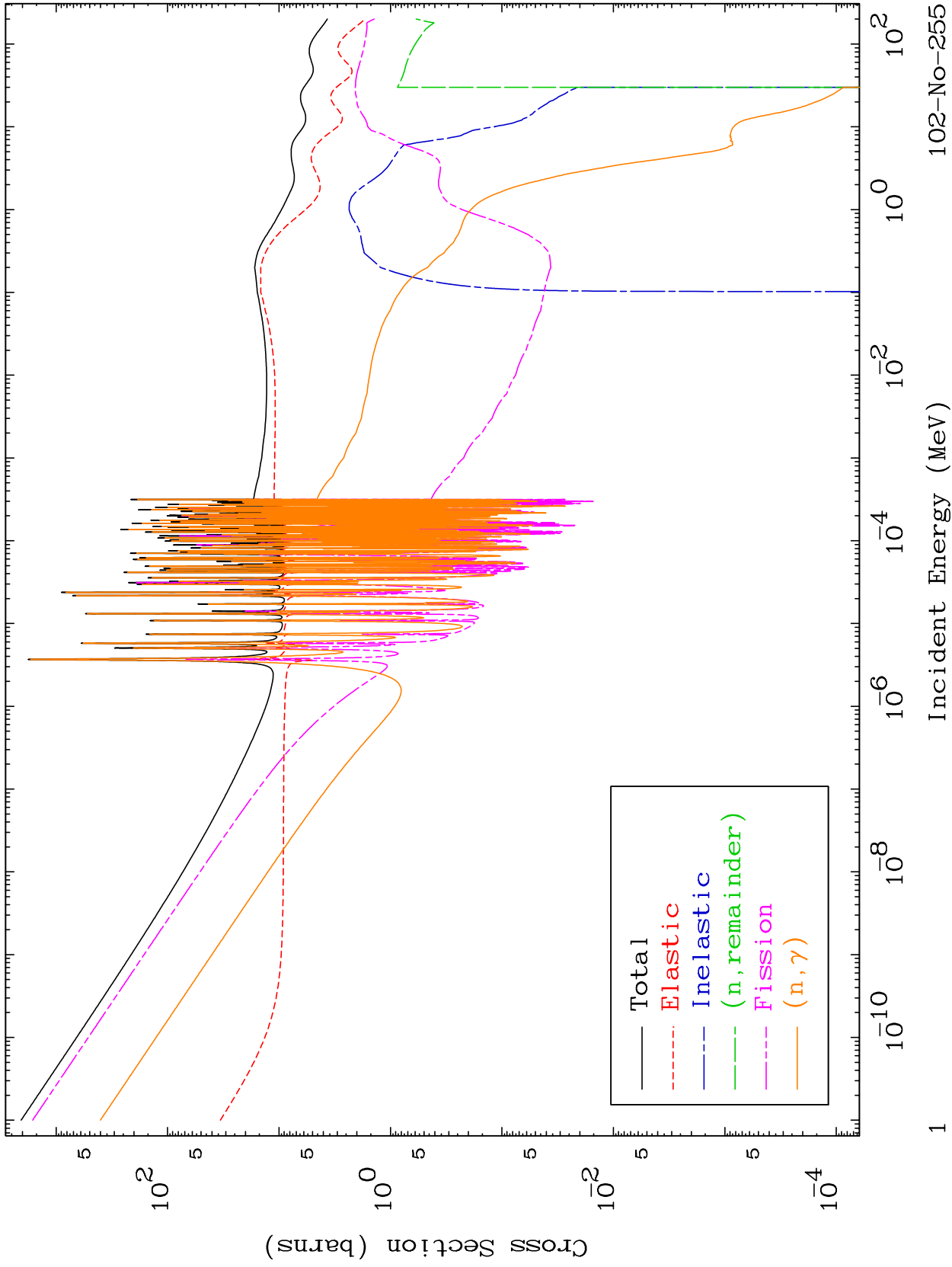
Press Mouse Button to Start

MAT 9973

Major

293 Kelvin Cross Sections

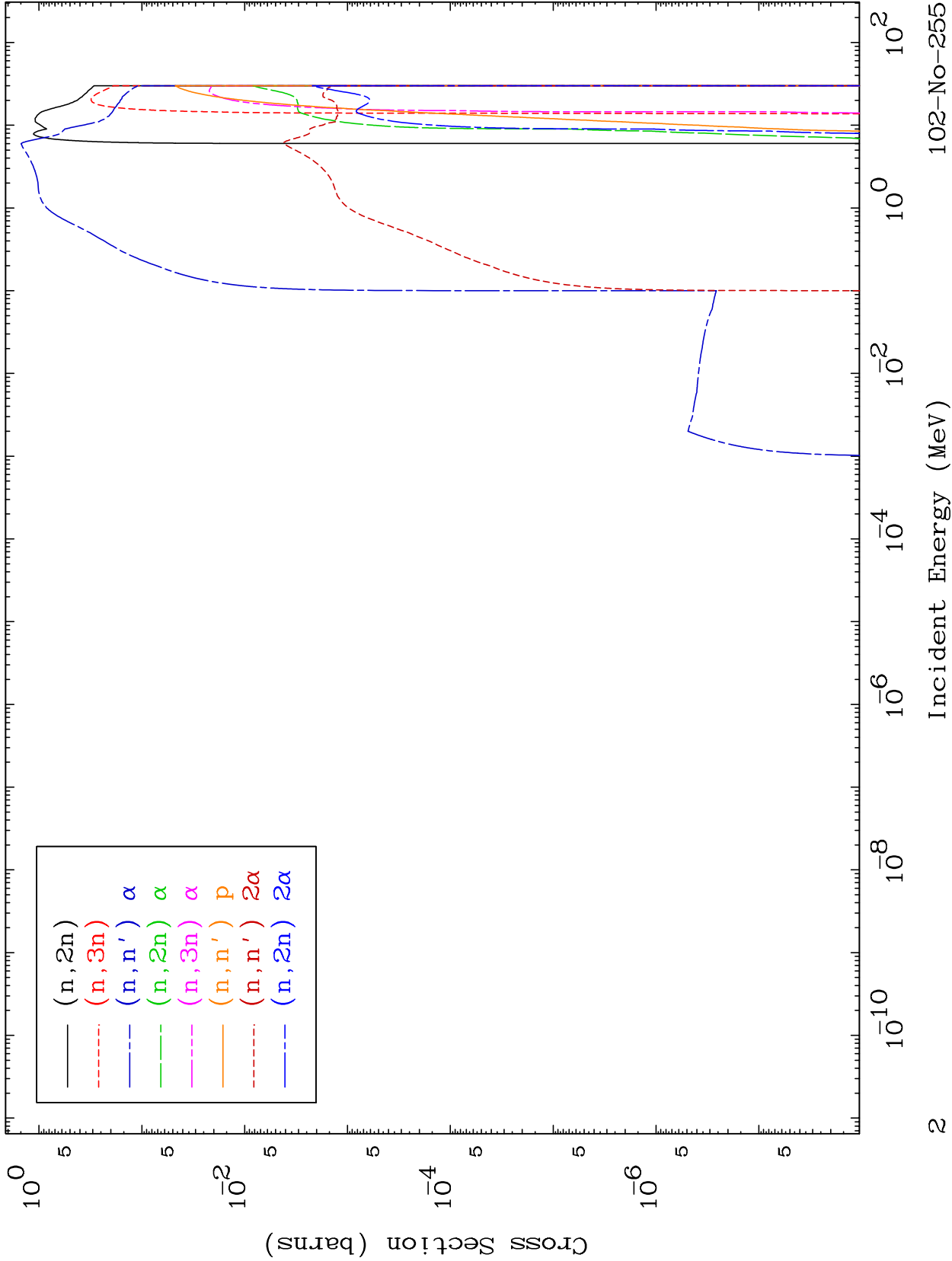
102-No-255



MAT 9973

Neutron Production
293 Kelvin Cross Sections

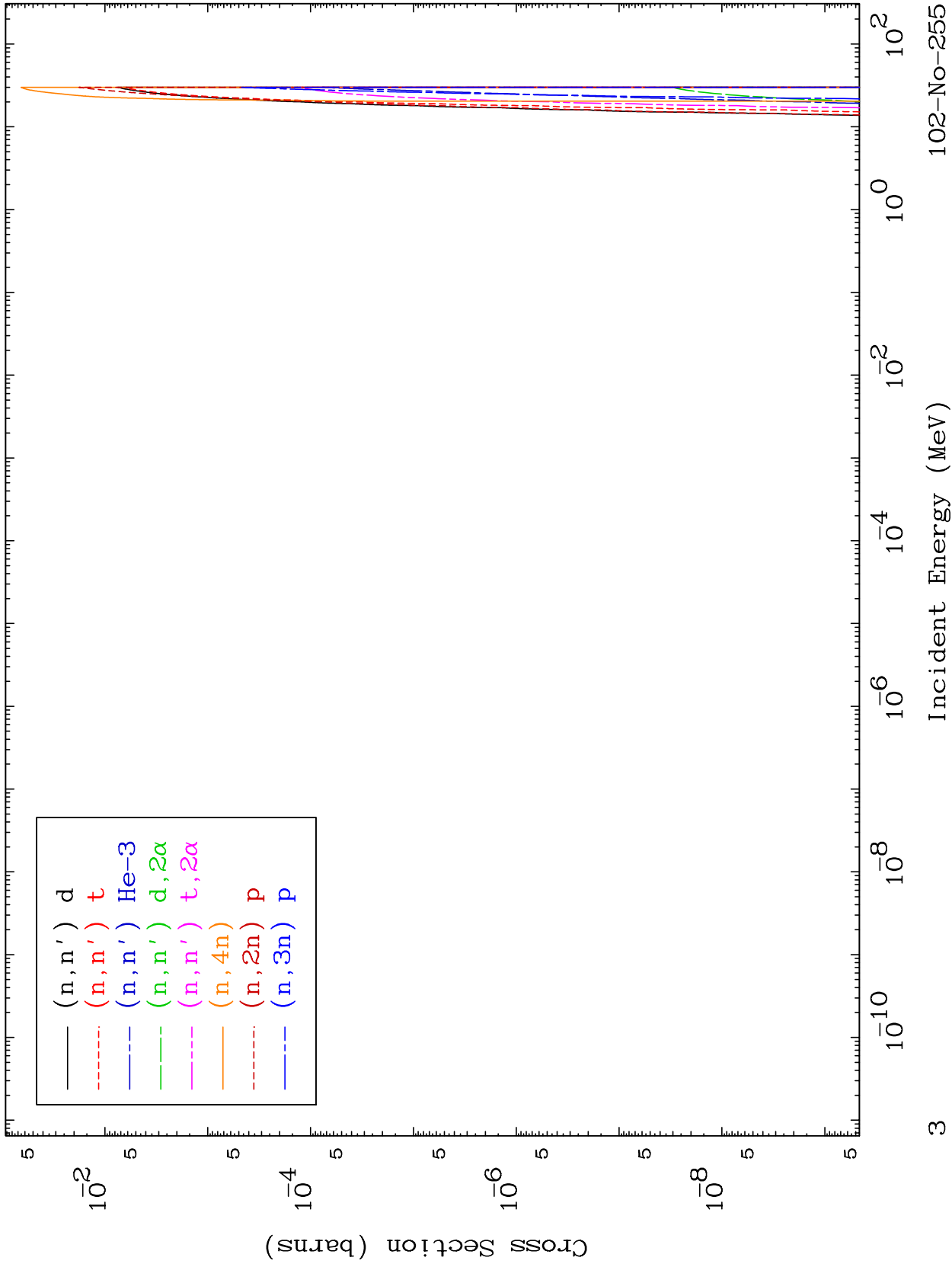
102-No-255



MAT 9973

Neutron Production
293 Kelvin Cross Sections

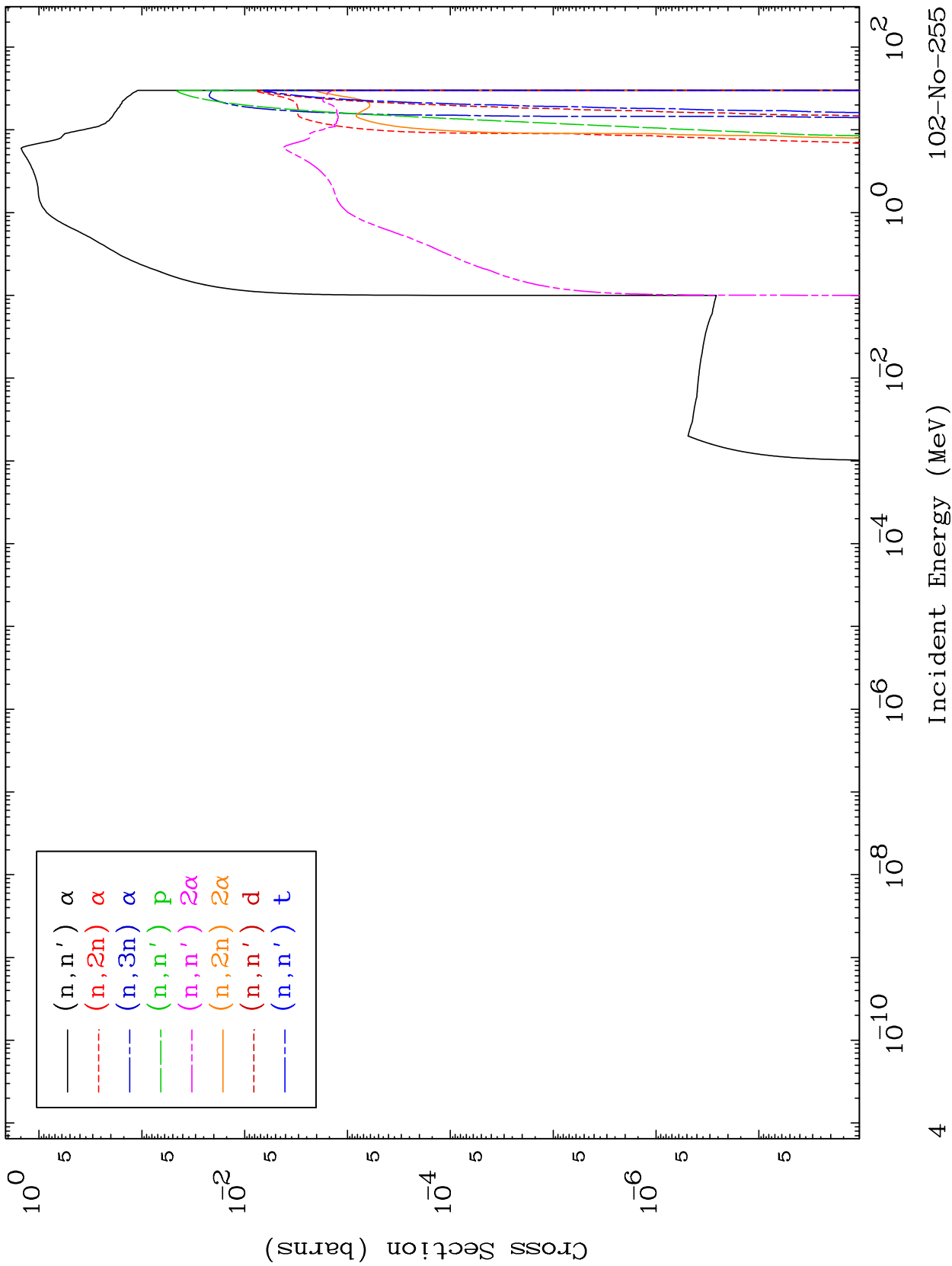
102-No-255



MAT 9973

Charged Particle
293 Kelvin Cross Sections

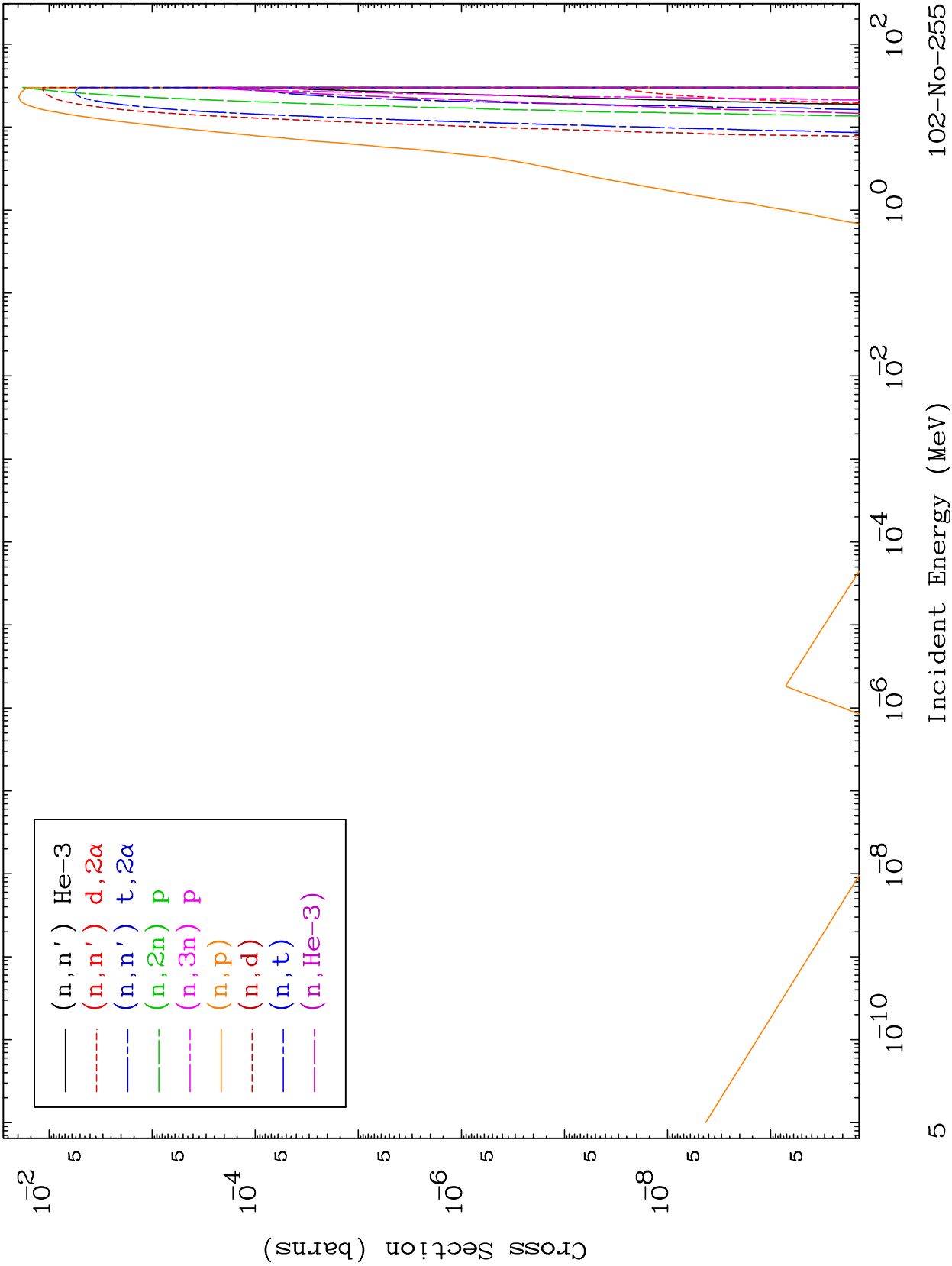
102-No-255



MAT 9973

Charged Particle
293 Kelvin Cross Sections

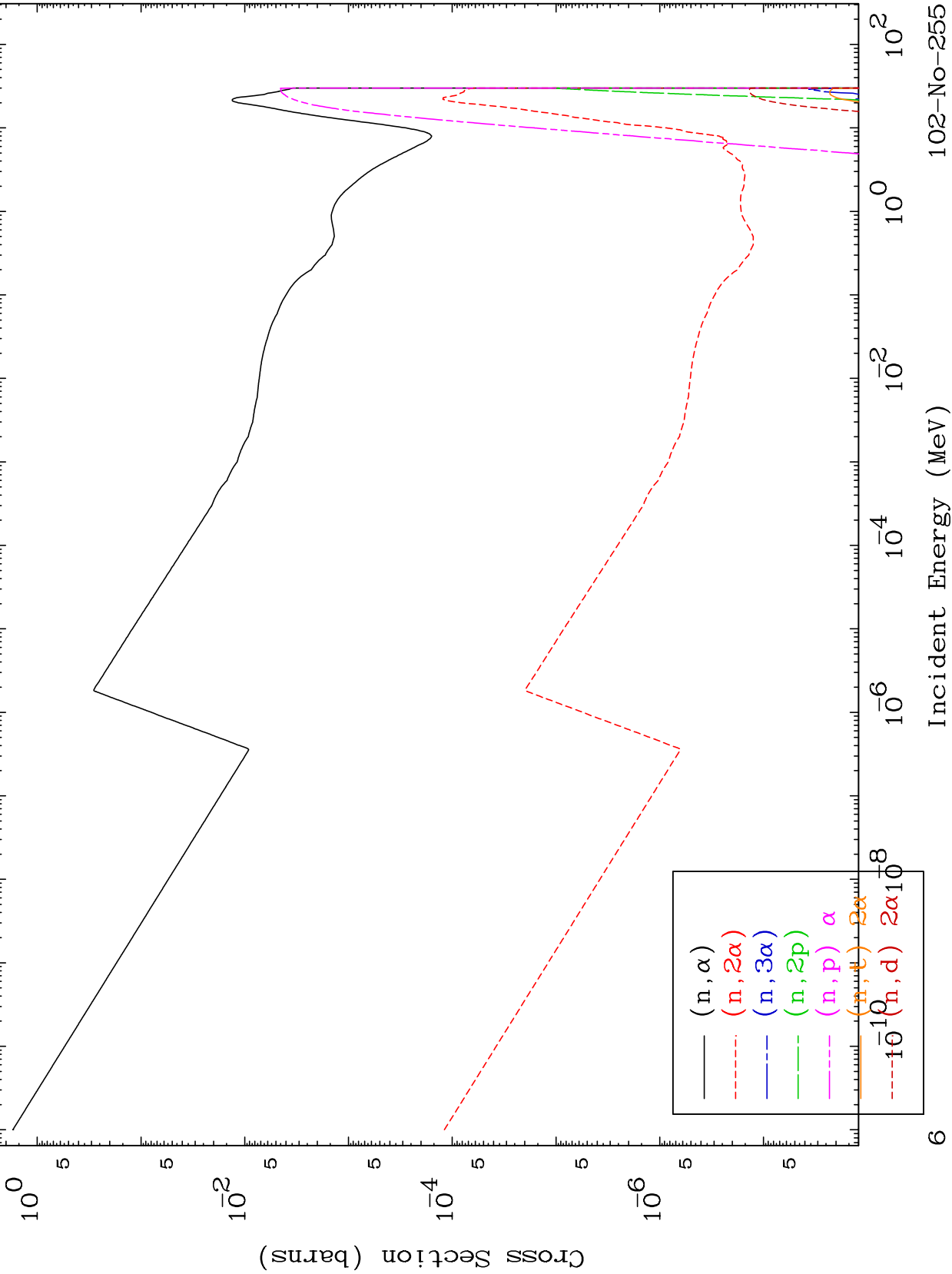
102-No-255



MAT 9973

Charged Particle
293 Kelvin Cross Sections

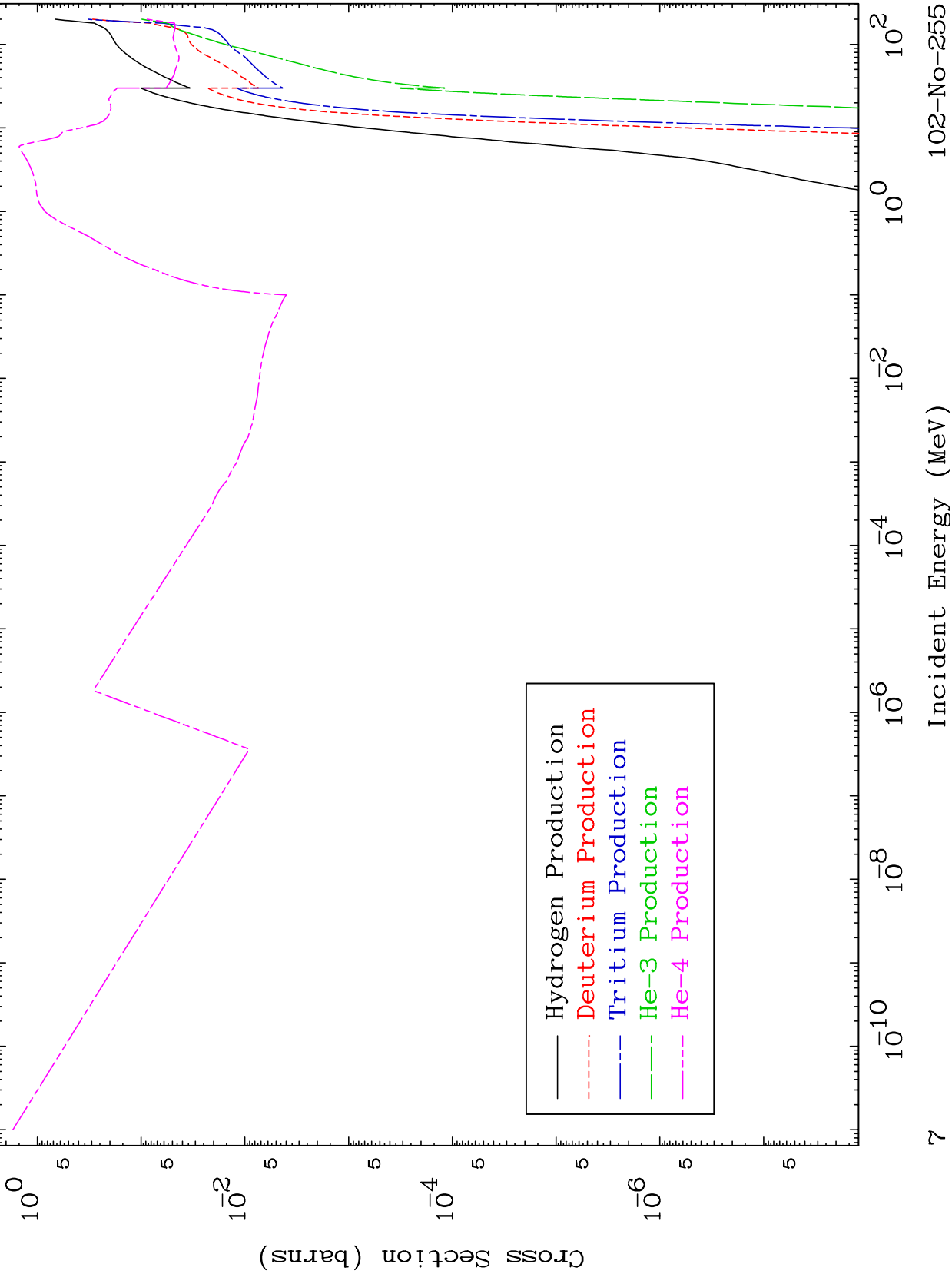
102-No-255



MAT 9973

Particle Production
293 Kelvin Cross Sections

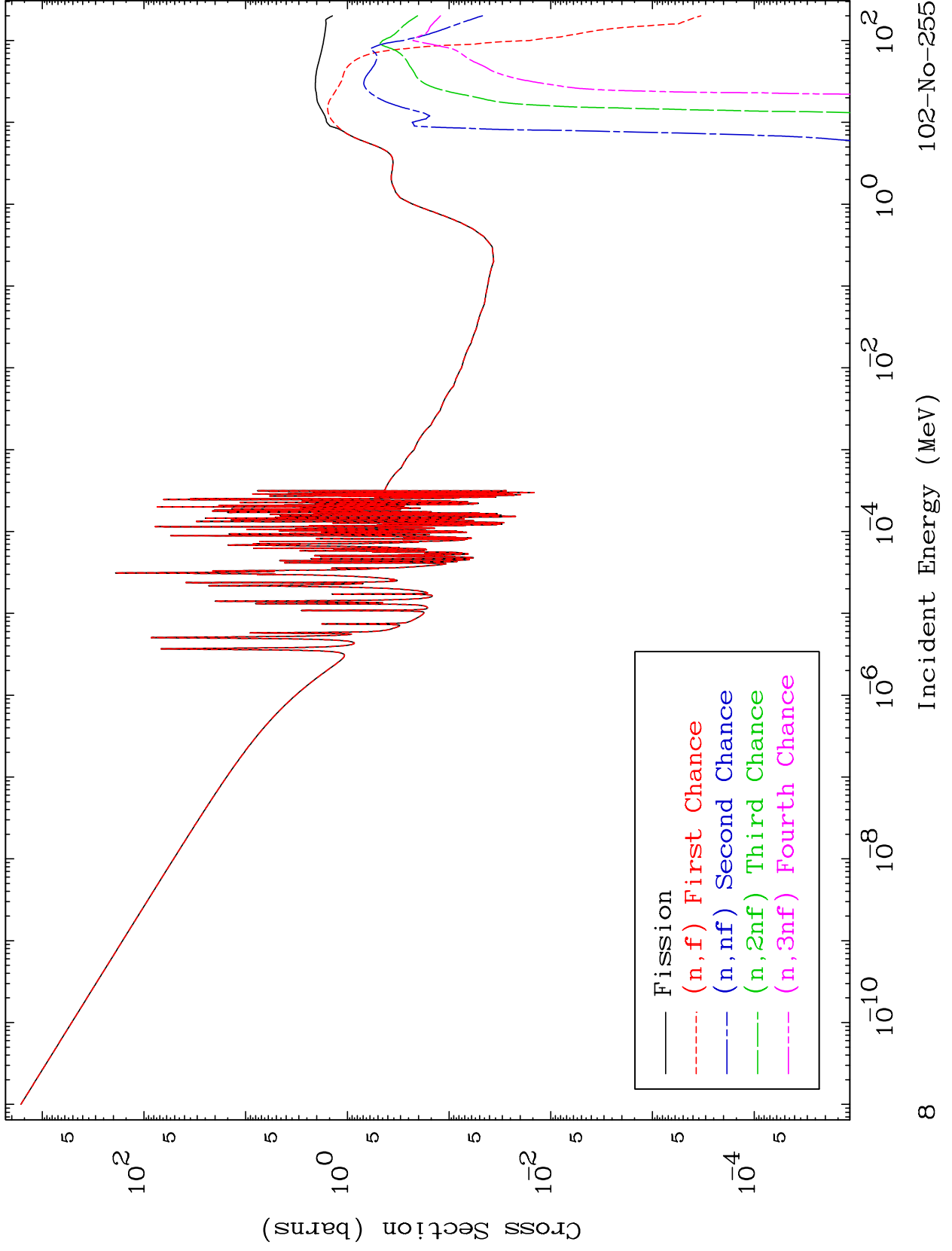
102-No-255



MAT 9973

Fission
293 Kelvin Cross Sections

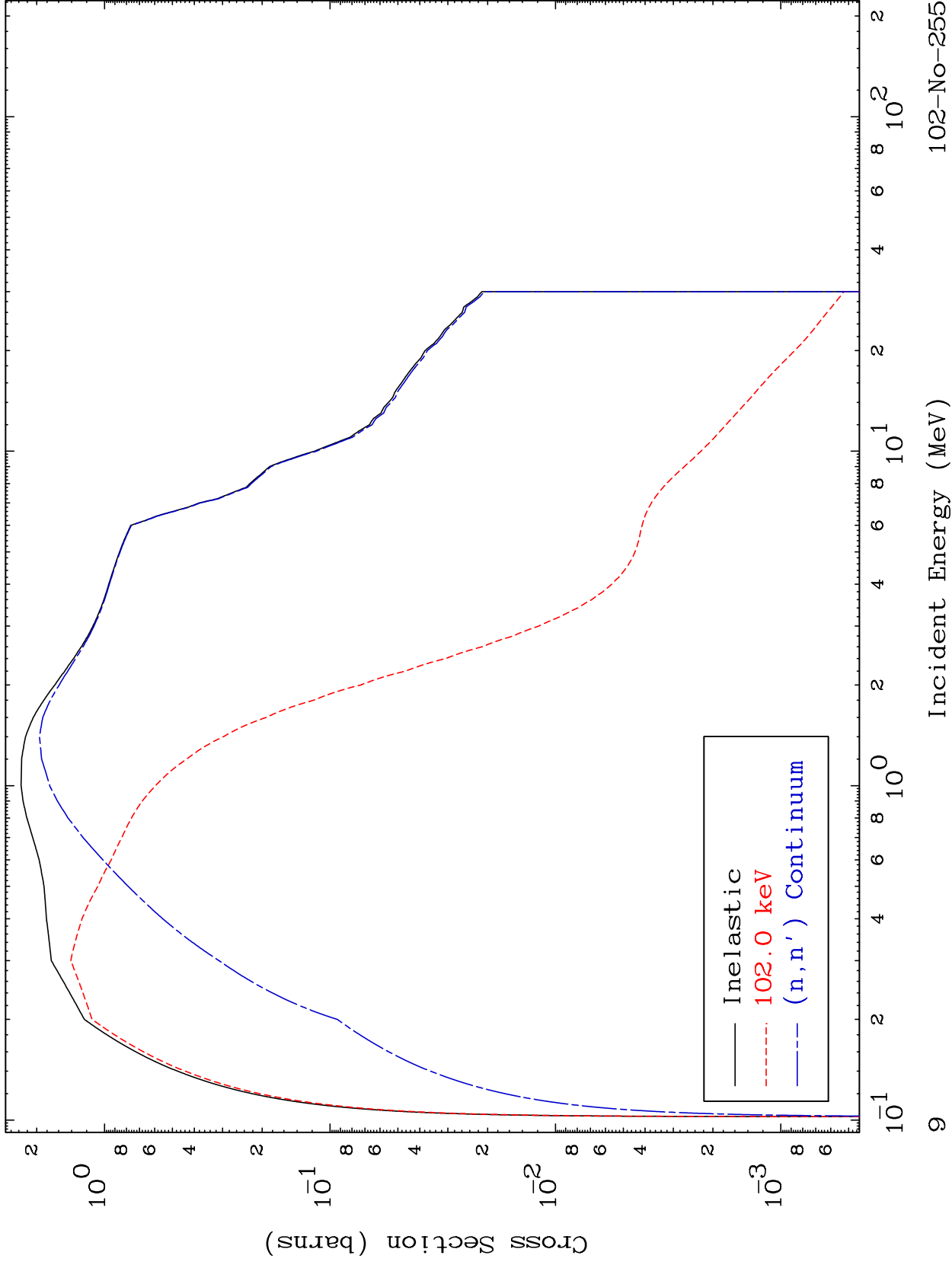
102-No-255



MAT 9973

(n,n') Level
293 Kelvin Cross Sections

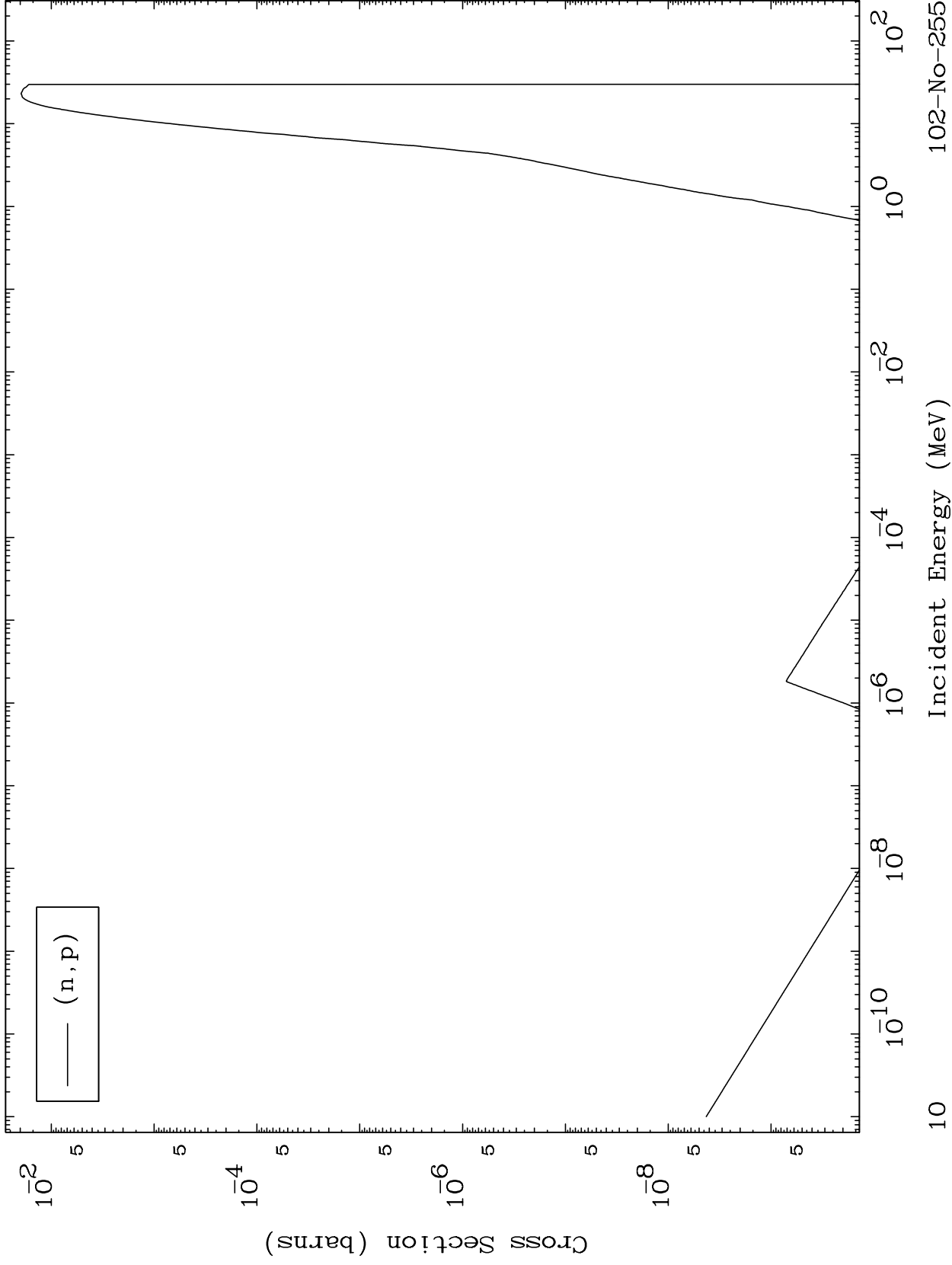
102-No-255



MAT 9973

(n,p) Levels
293 Kelvin Cross Sections

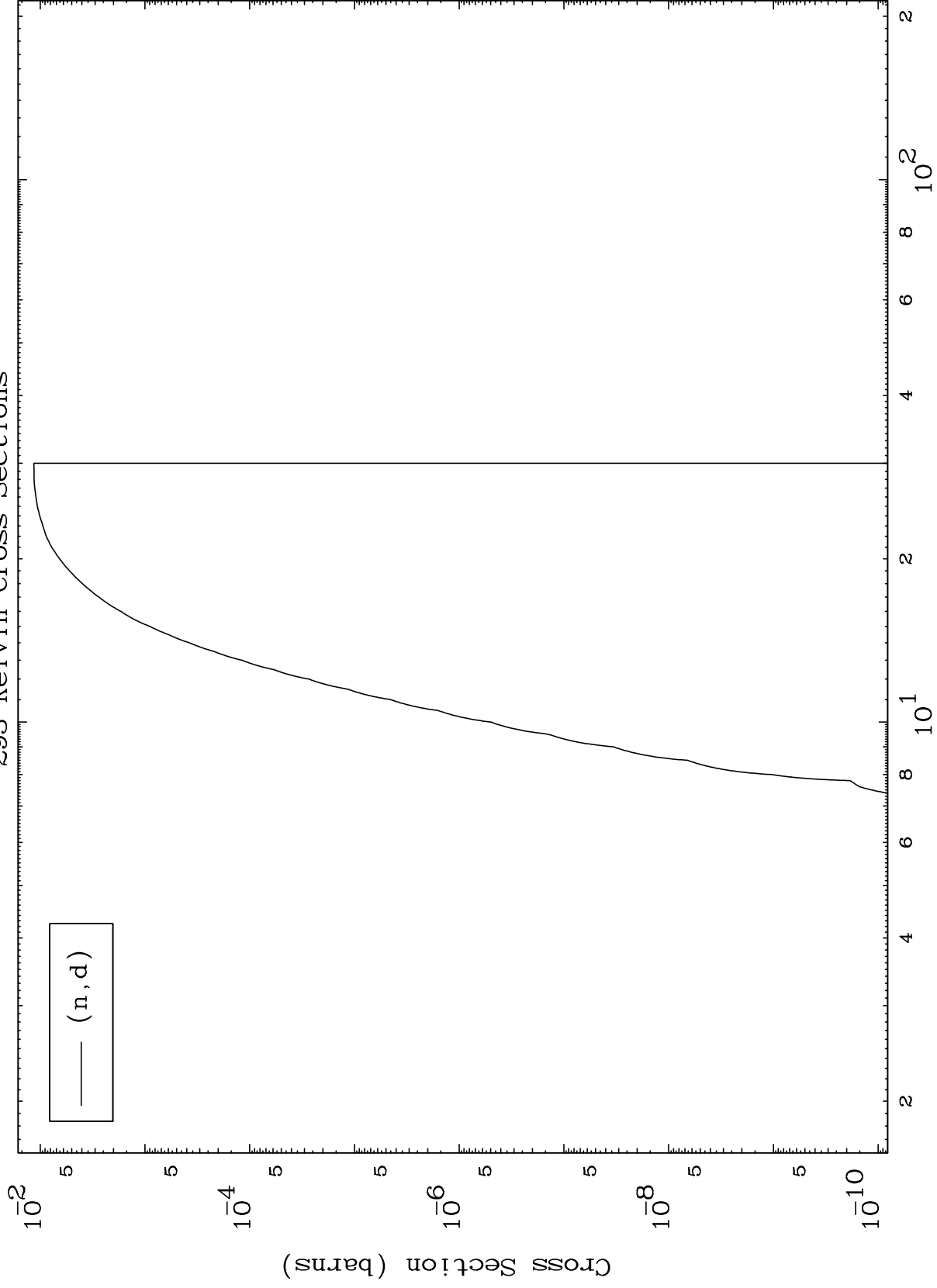
102-No-255



MAT 9973

(n,d) Levels
293 Kelvin Cross Sections

102-No-255



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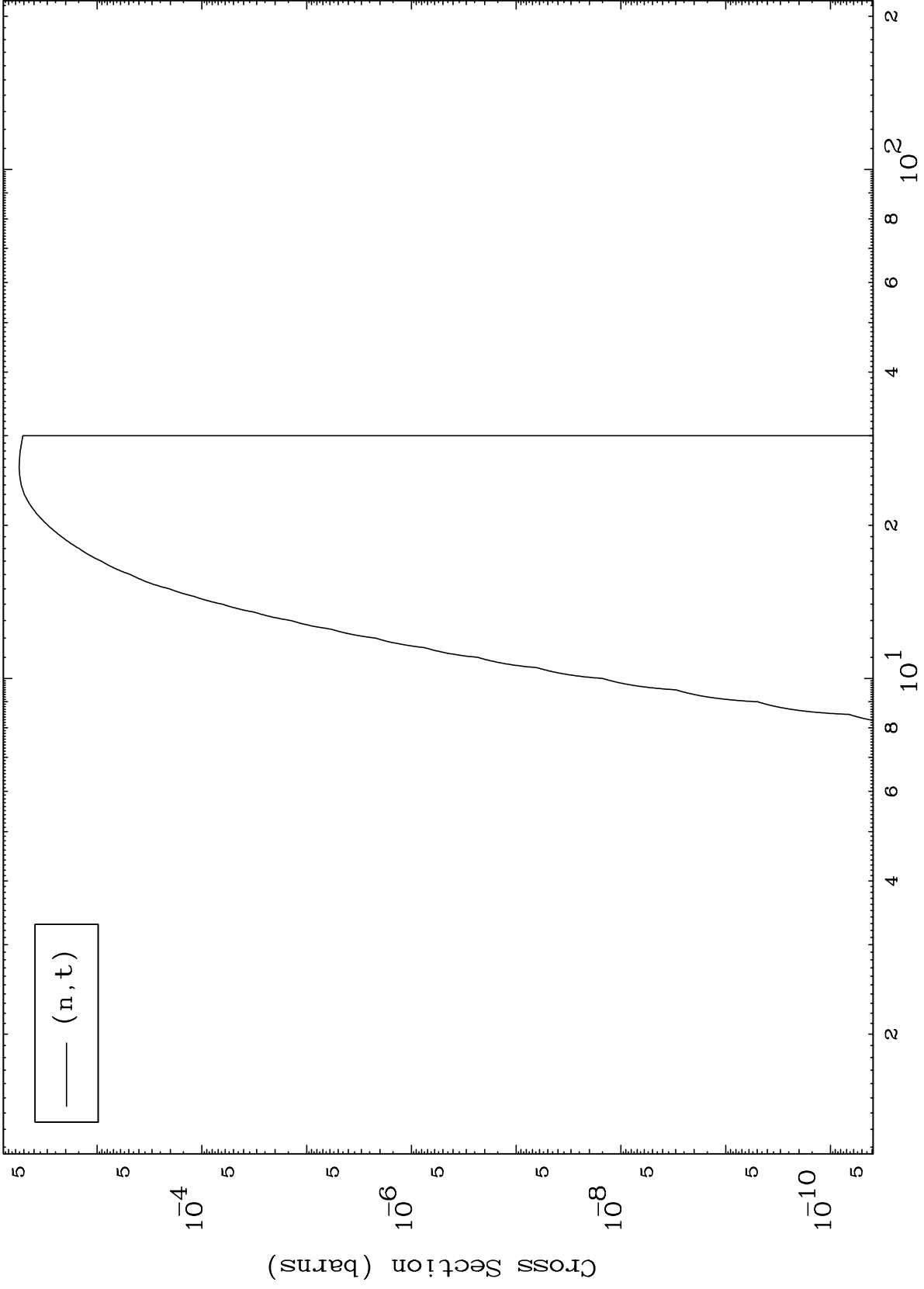
Incident Energy (MeV)

102-No-255

MAT 9973

(n,t) Levels
293 Kelvin Cross Sections

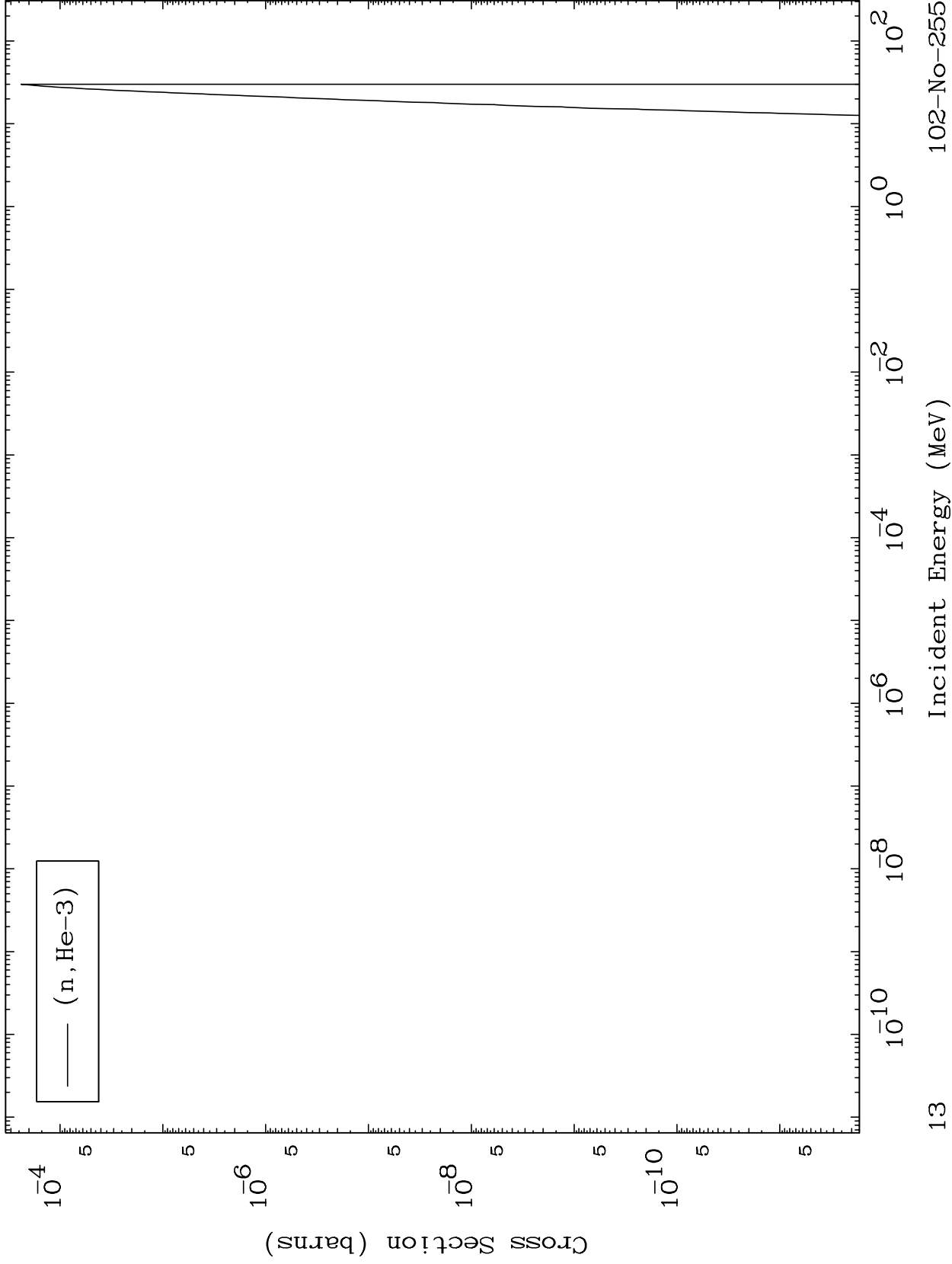
102-No-255



MAT 9973

(n,He3) Levels
293 Kelvin Cross Sections

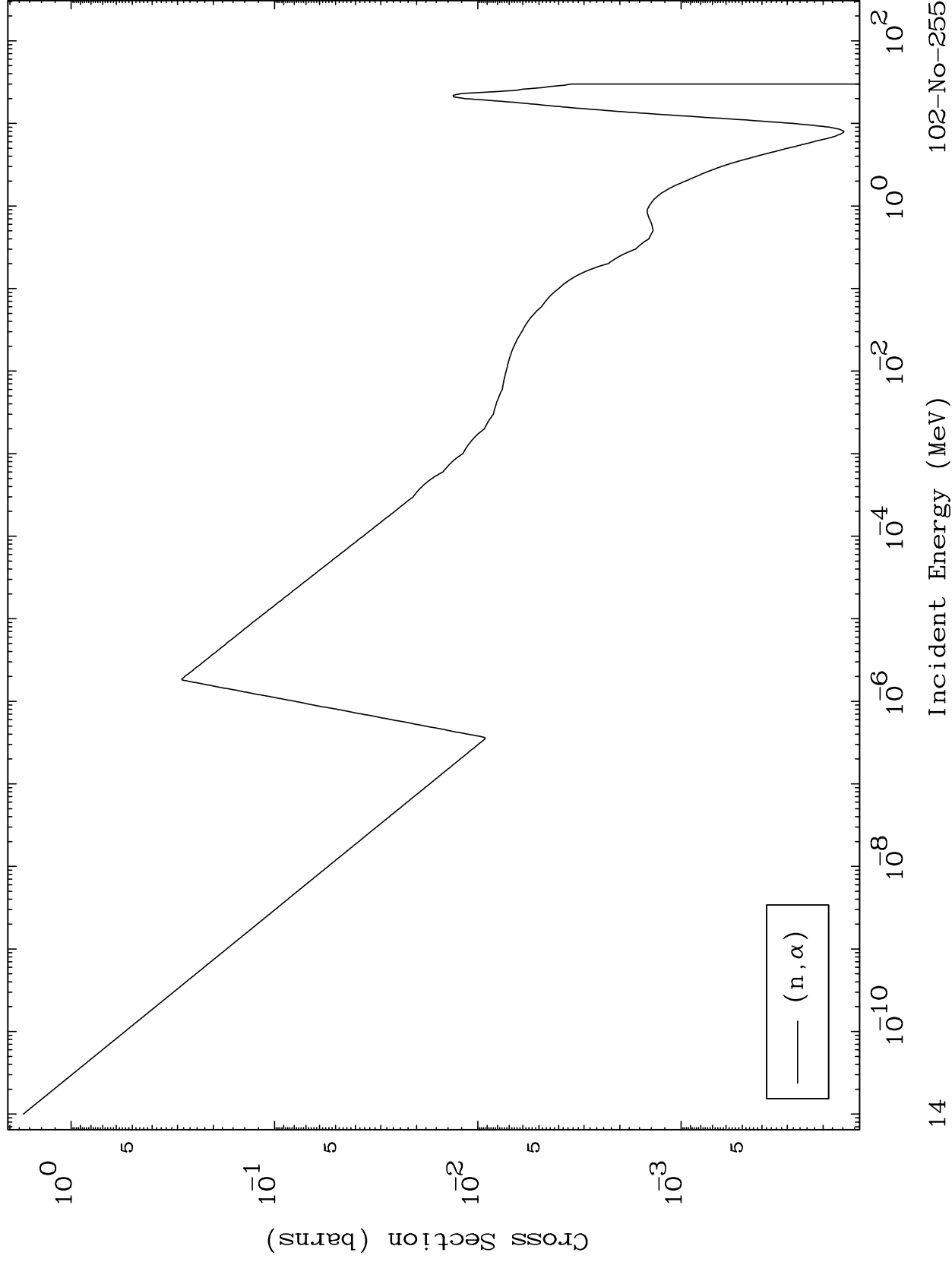
102-No-255

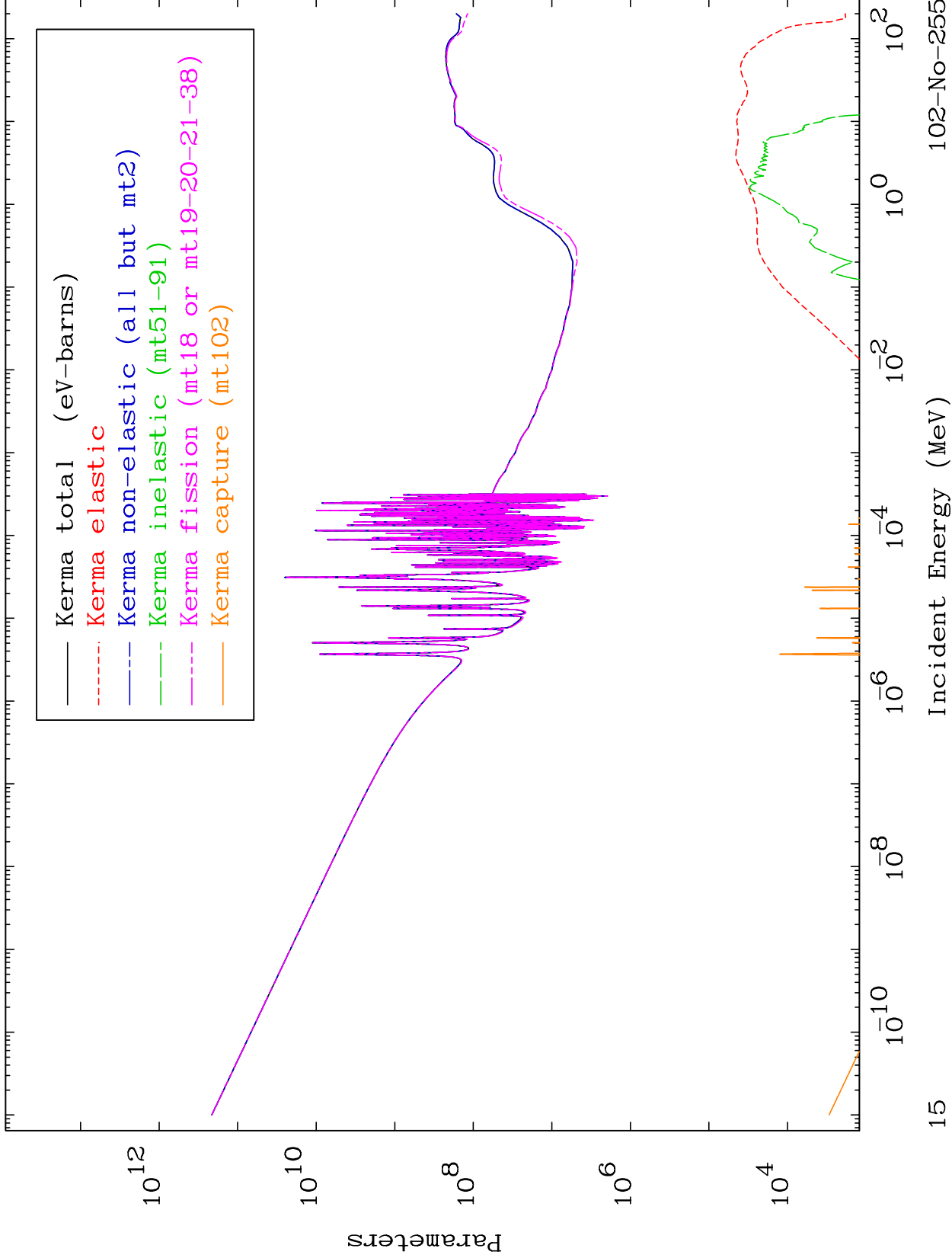


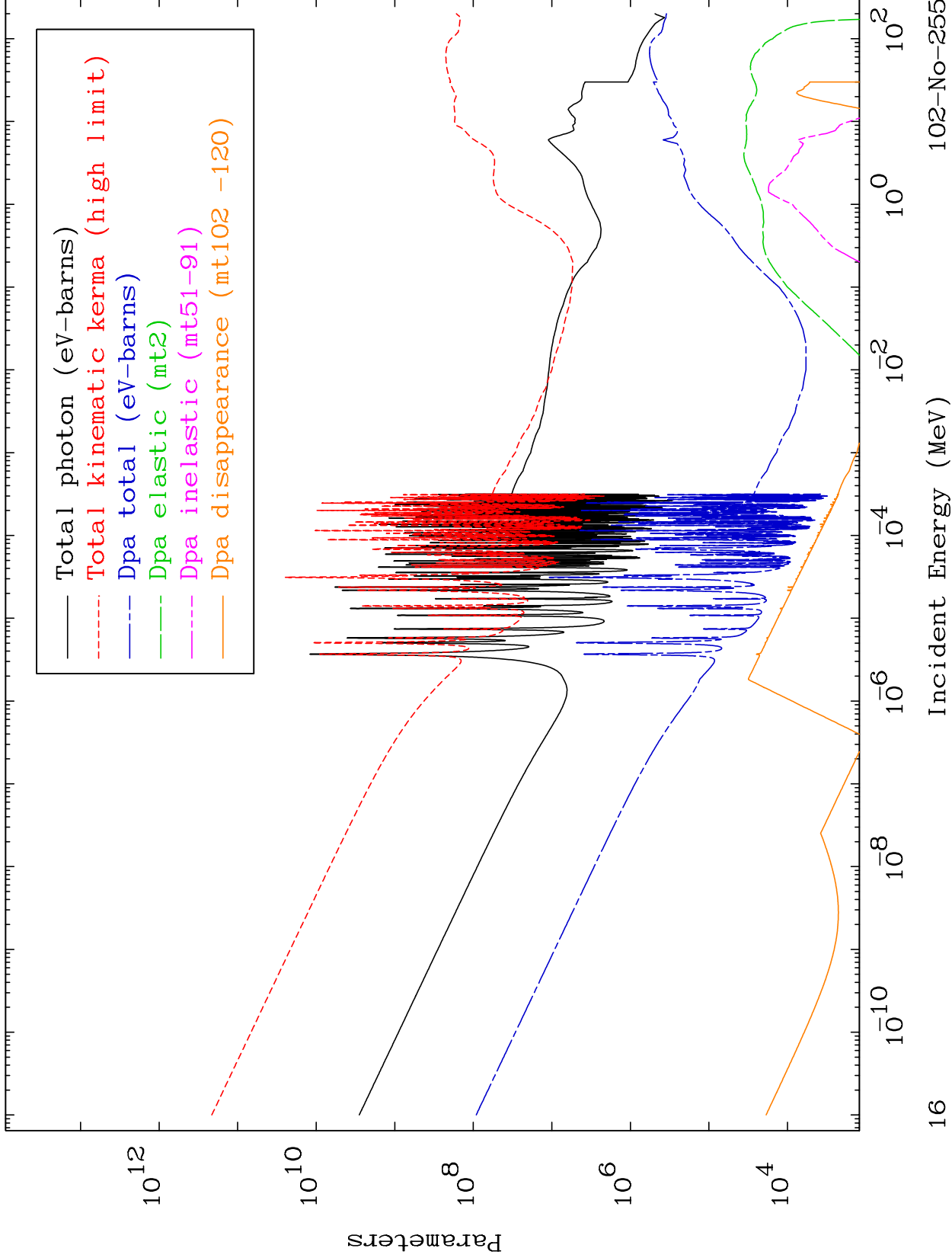
MAT 9973

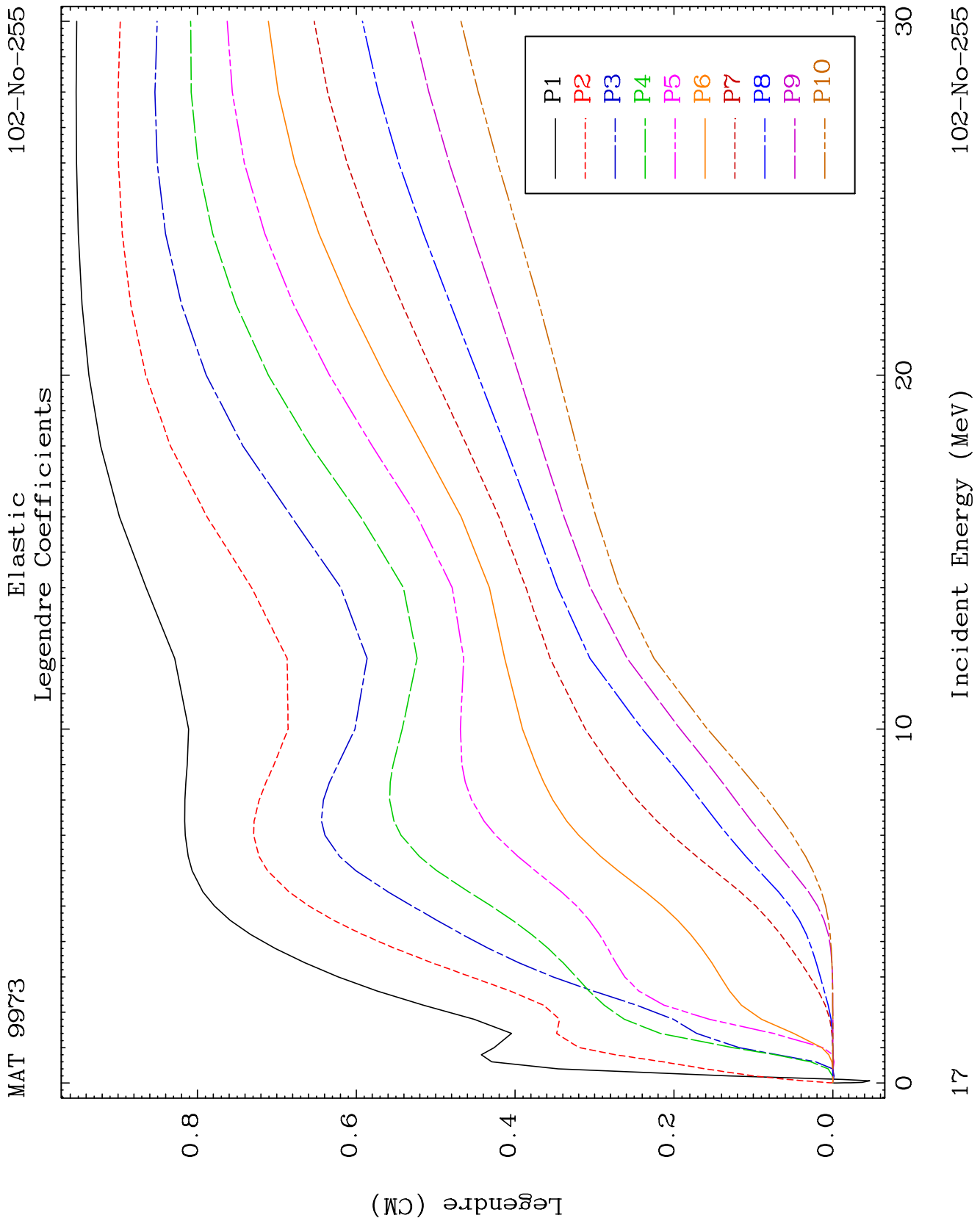
(n, α) Levels
293 Kelvin Cross Sections

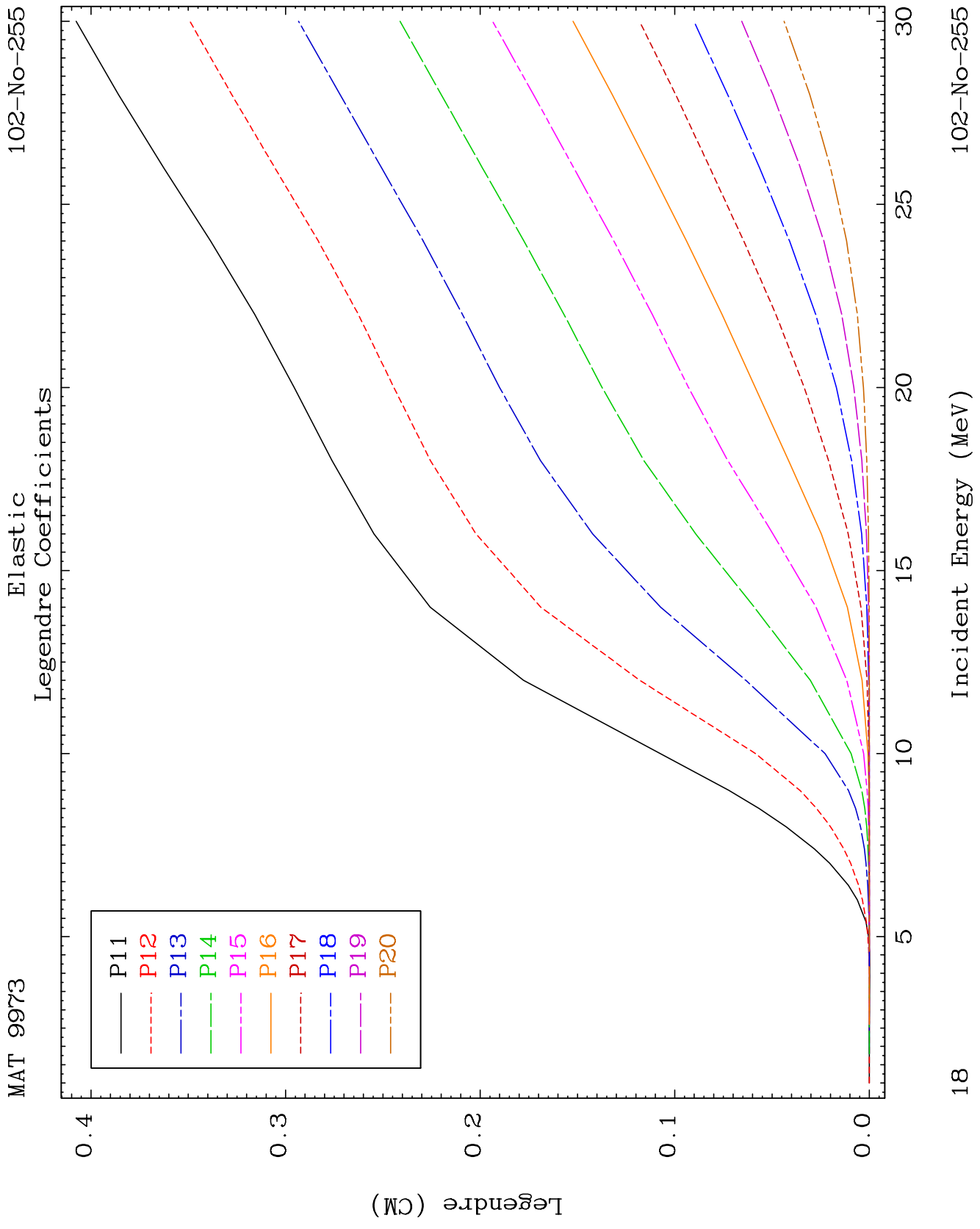
102-No-255











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Elastic Legendre Coefficients

102-No-255

18

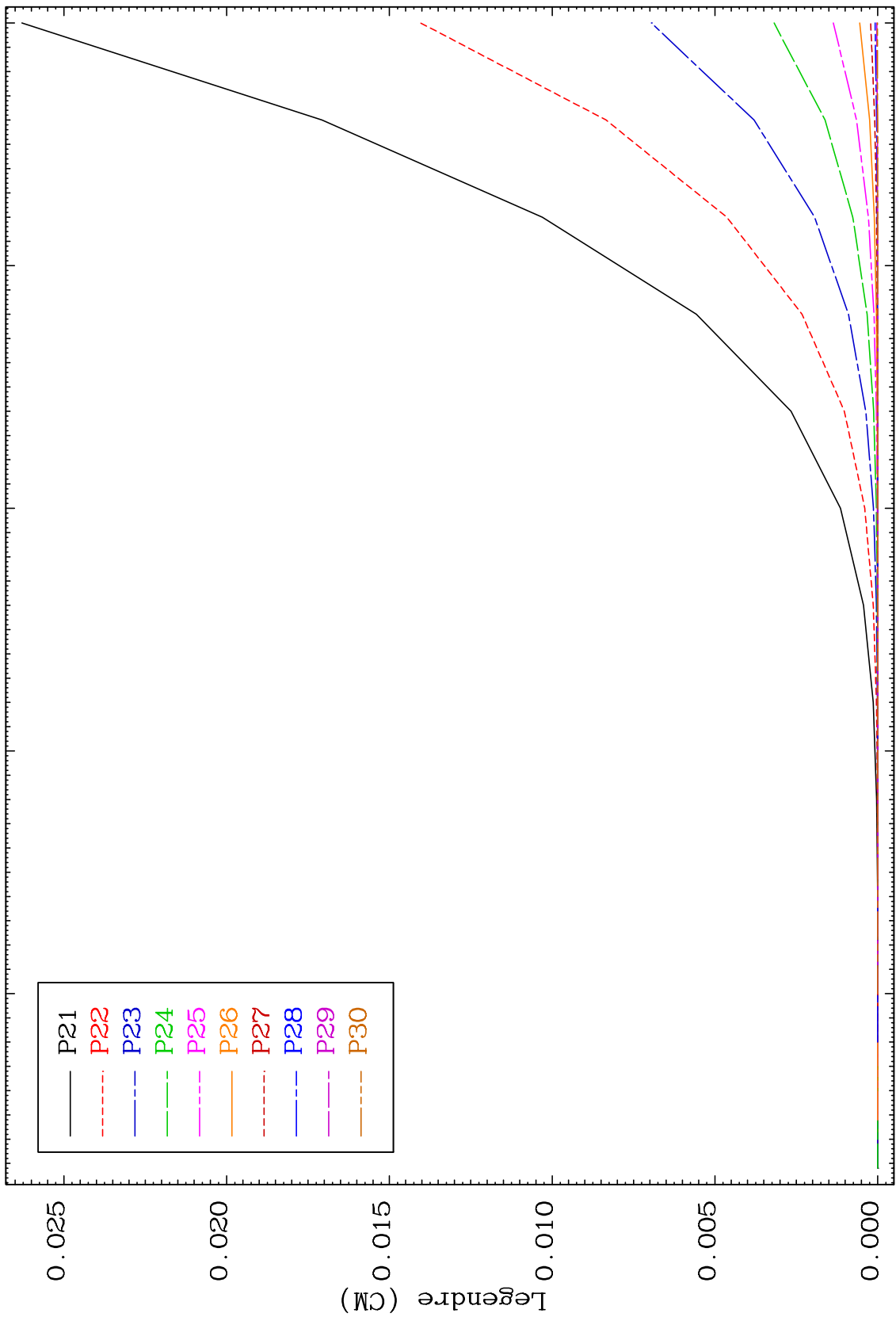
Incident Energy (MeV)

102-No-255

MAT 9973

Elastic Legendre Coefficients

102-No-255



19

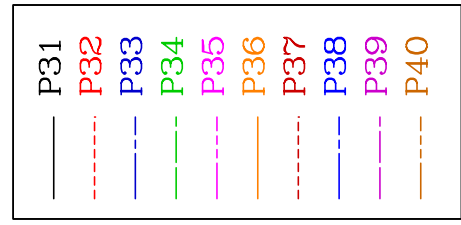
Incident Energy (MeV)

102-No-255

MAT 9973

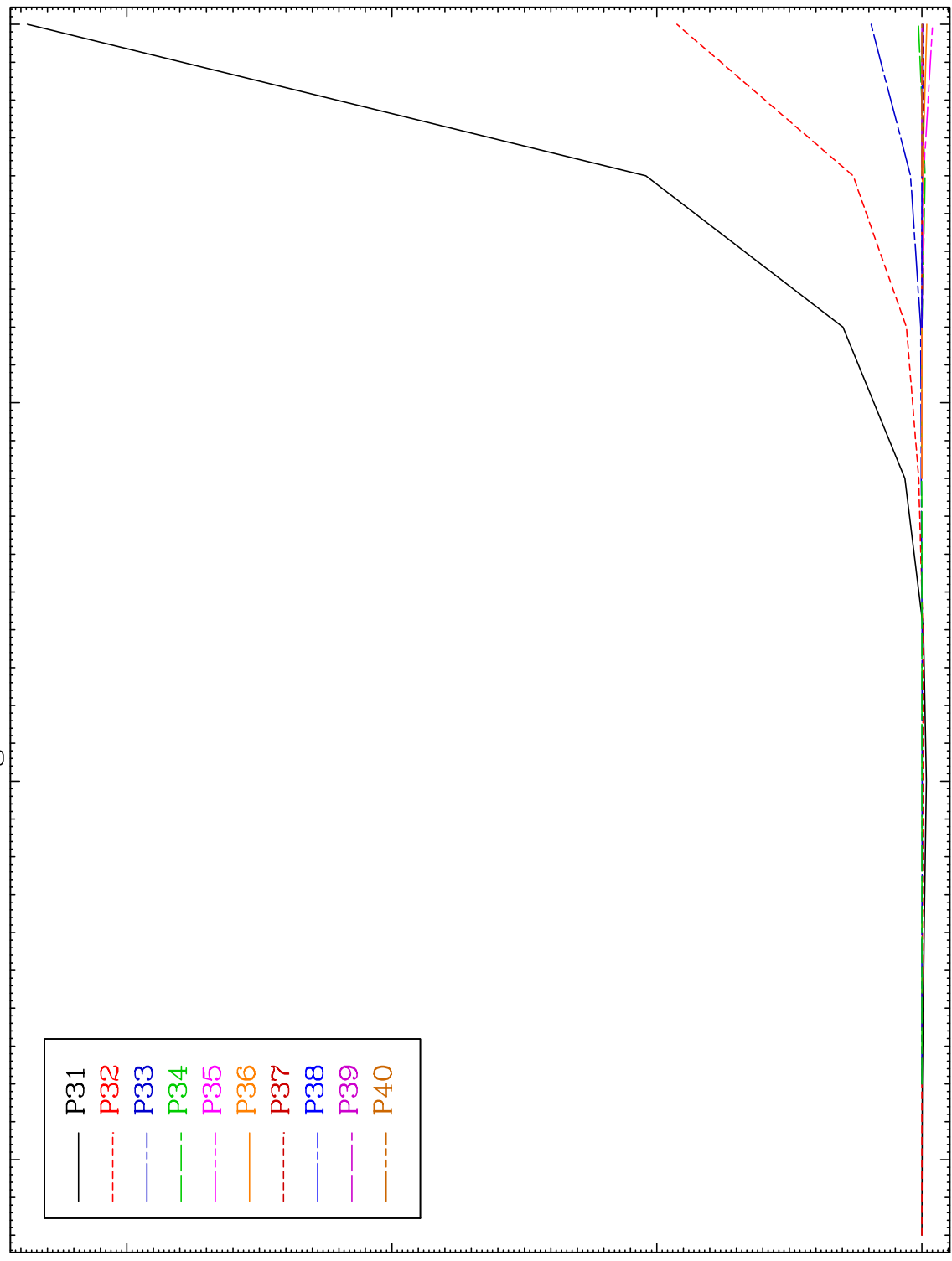
Elastic Legendre Coefficients

102-No-255



$\times 10^{-6}$

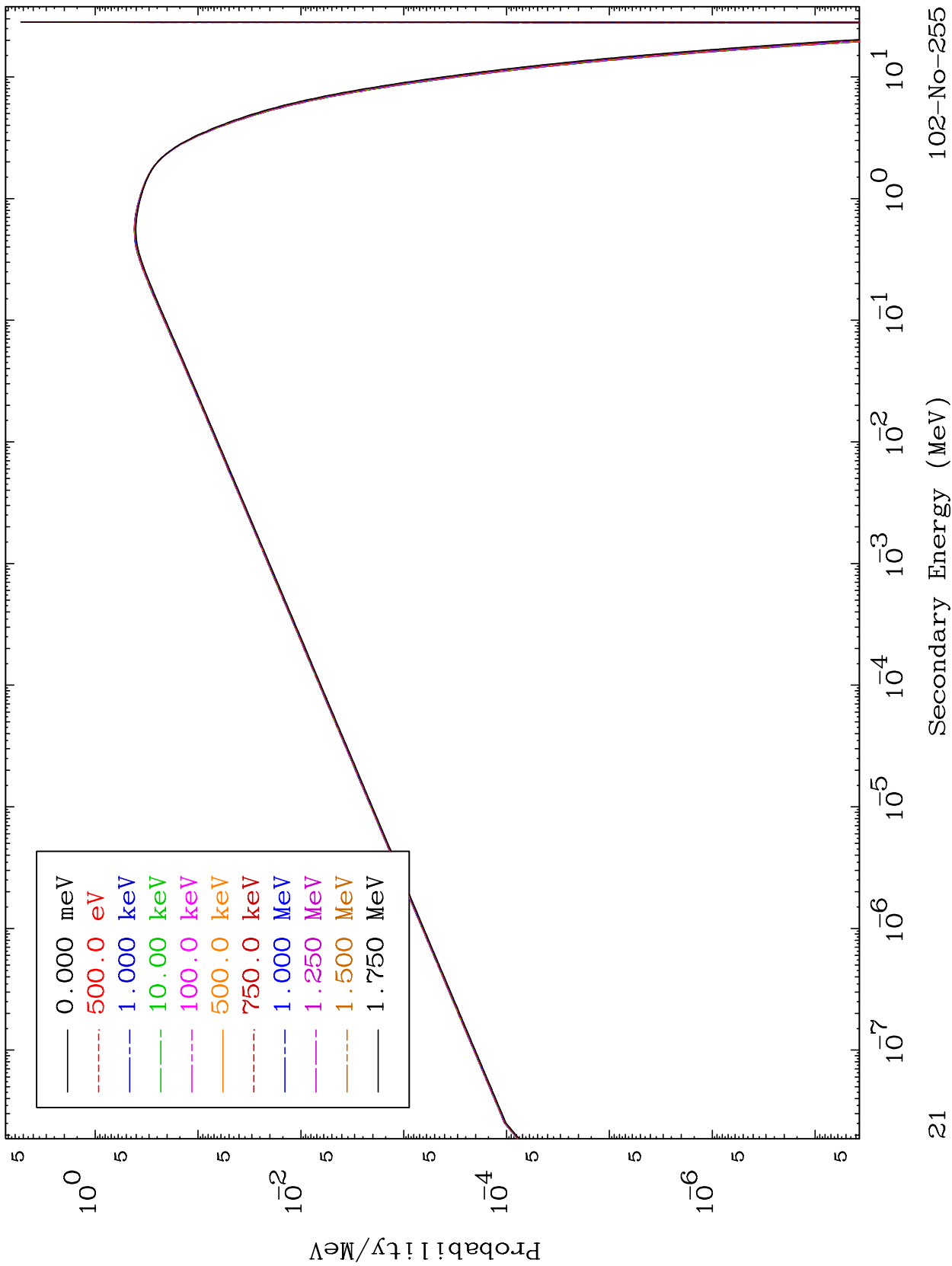
Legendre (CM)



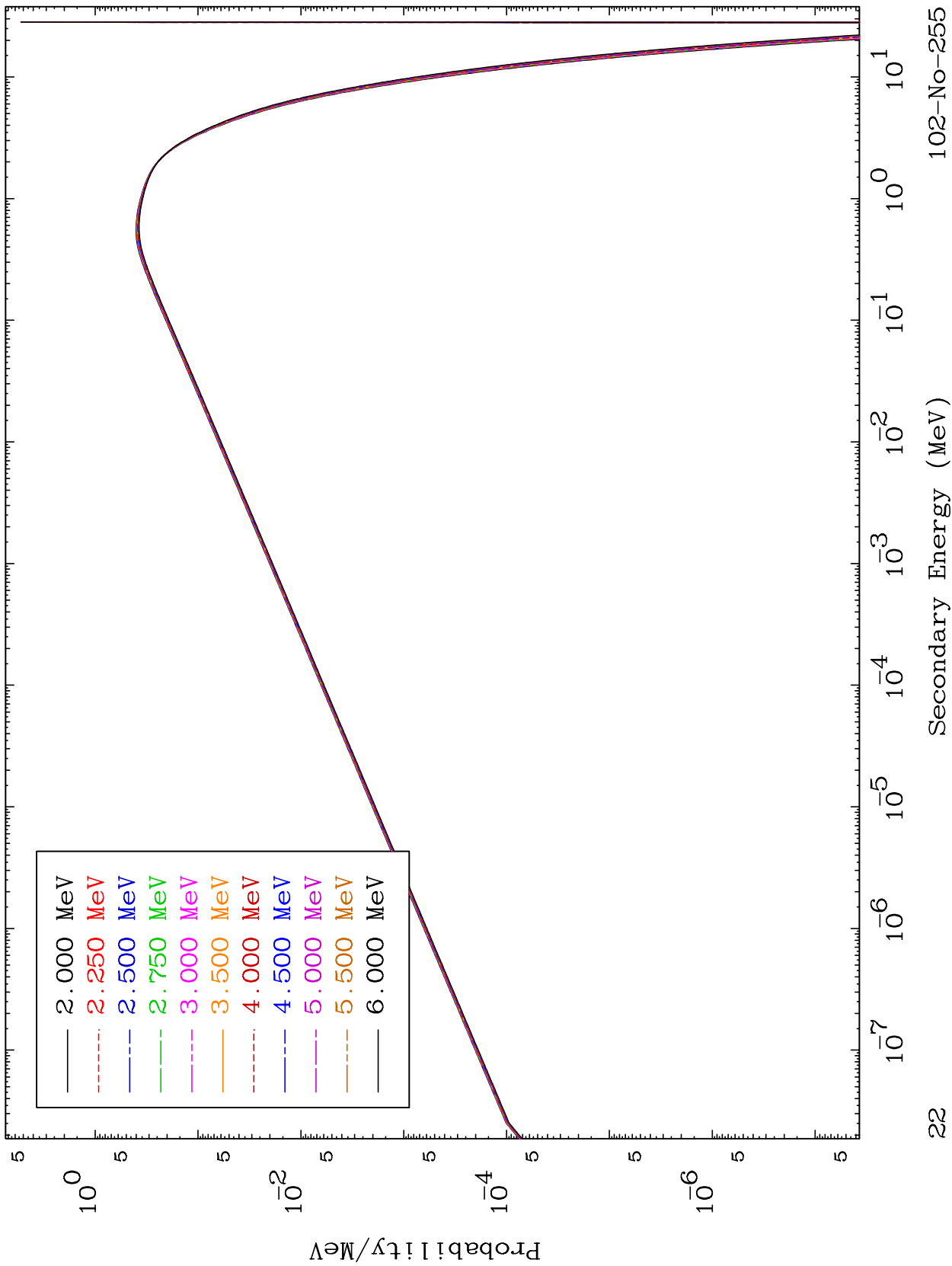
20

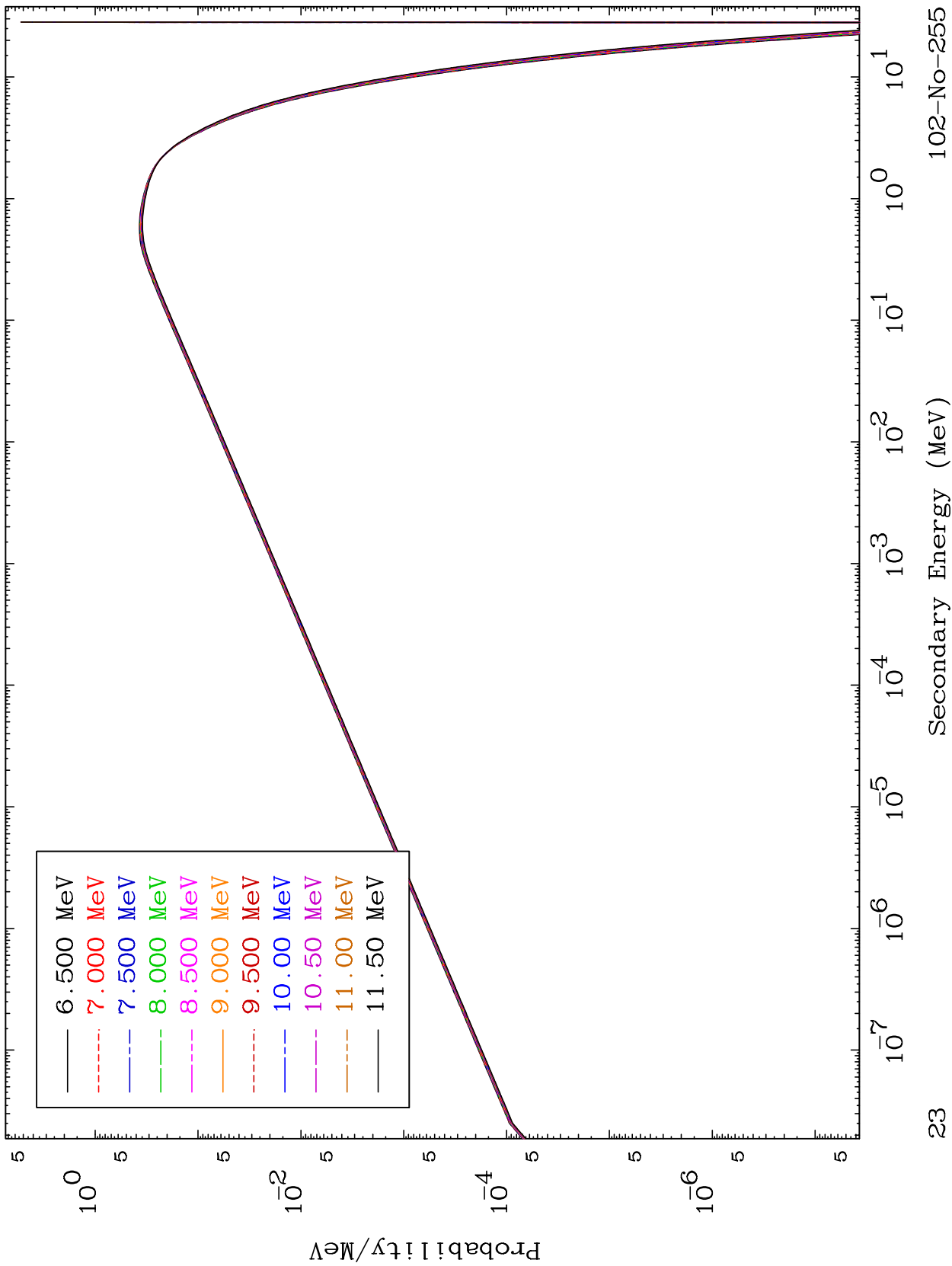
Incident Energy (MeV)

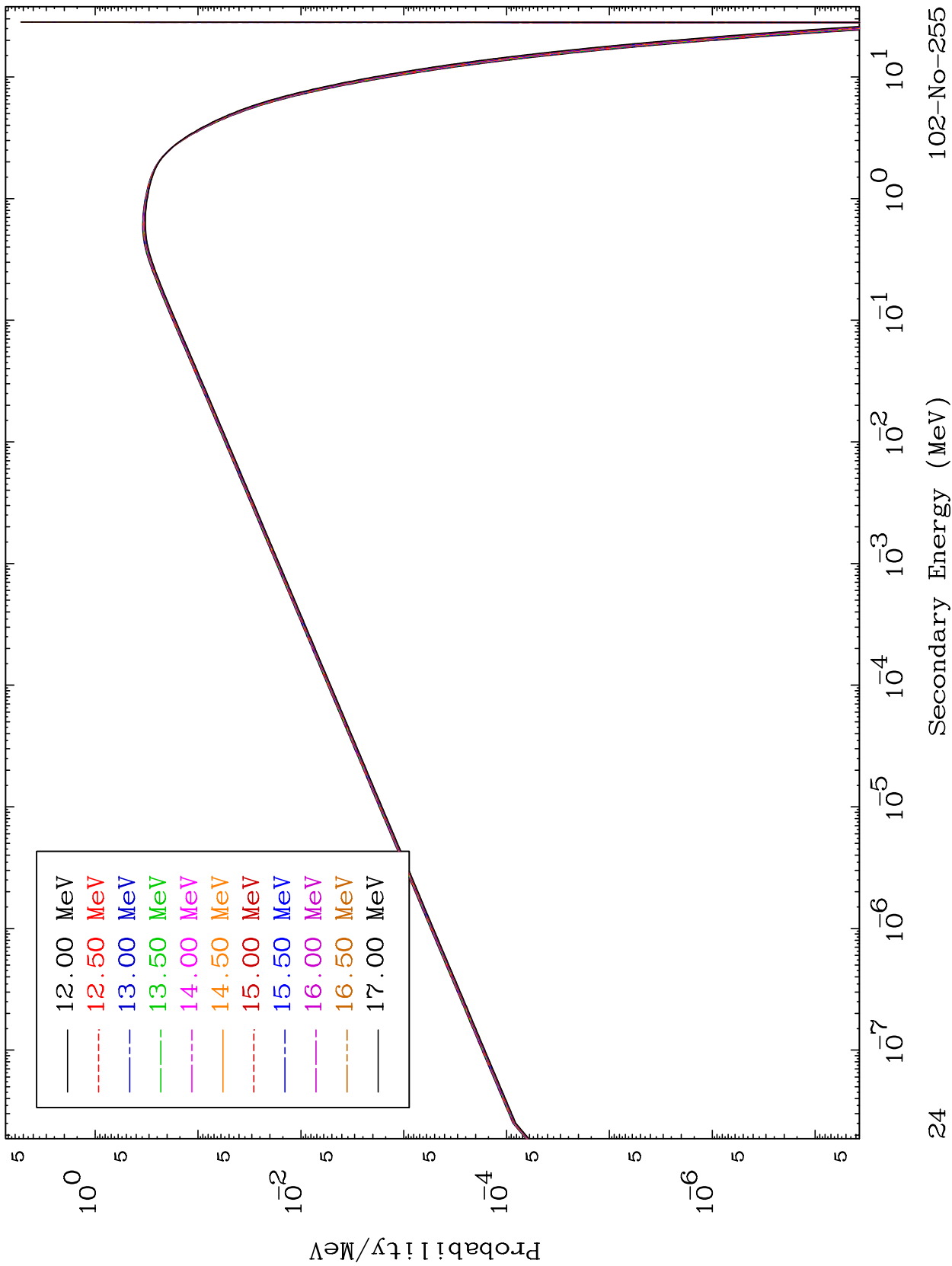
102-No-255

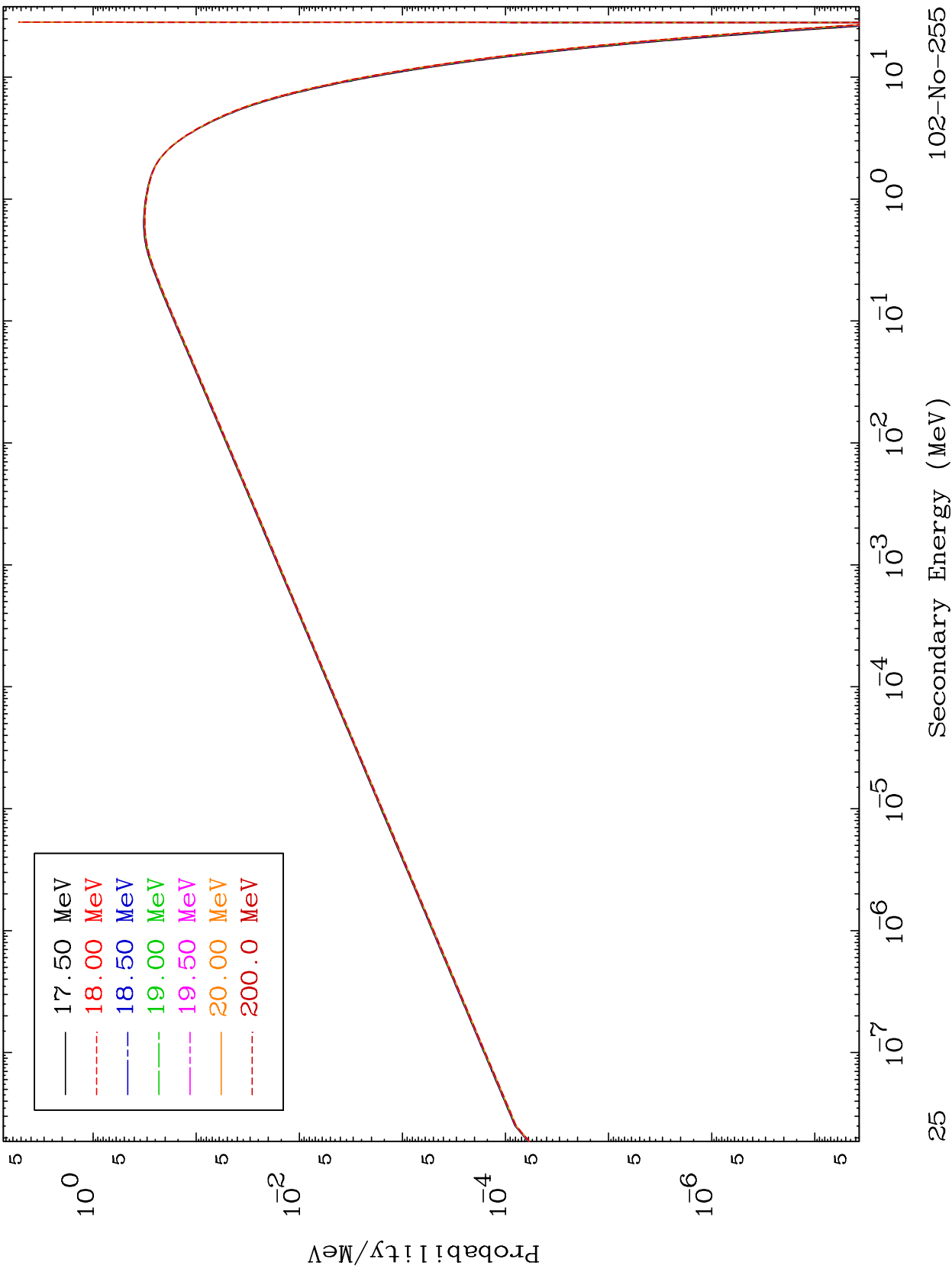


Fission
Energy Distribution







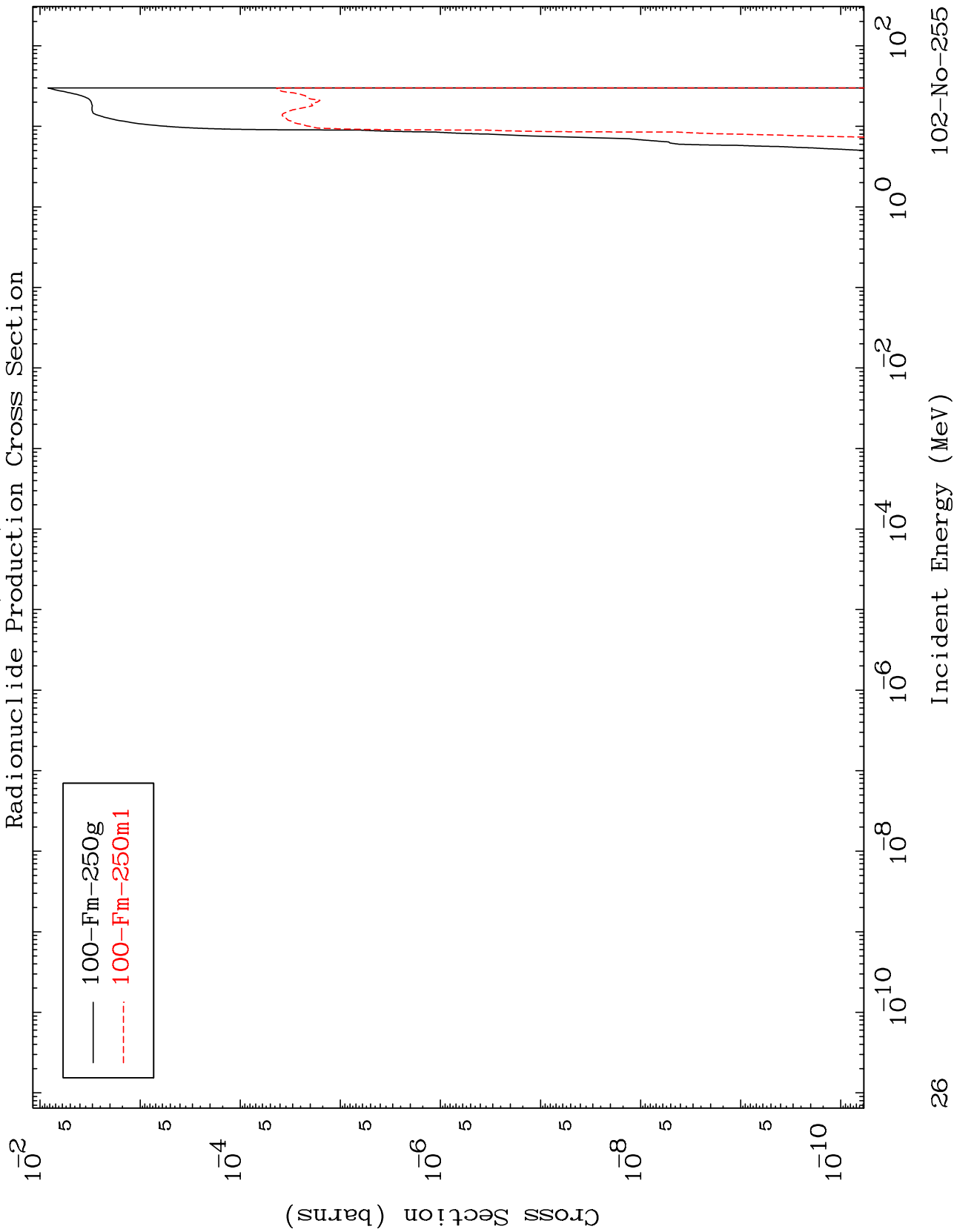


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(n,2n) α

102-No-255

Radionuclide Production Cross Section

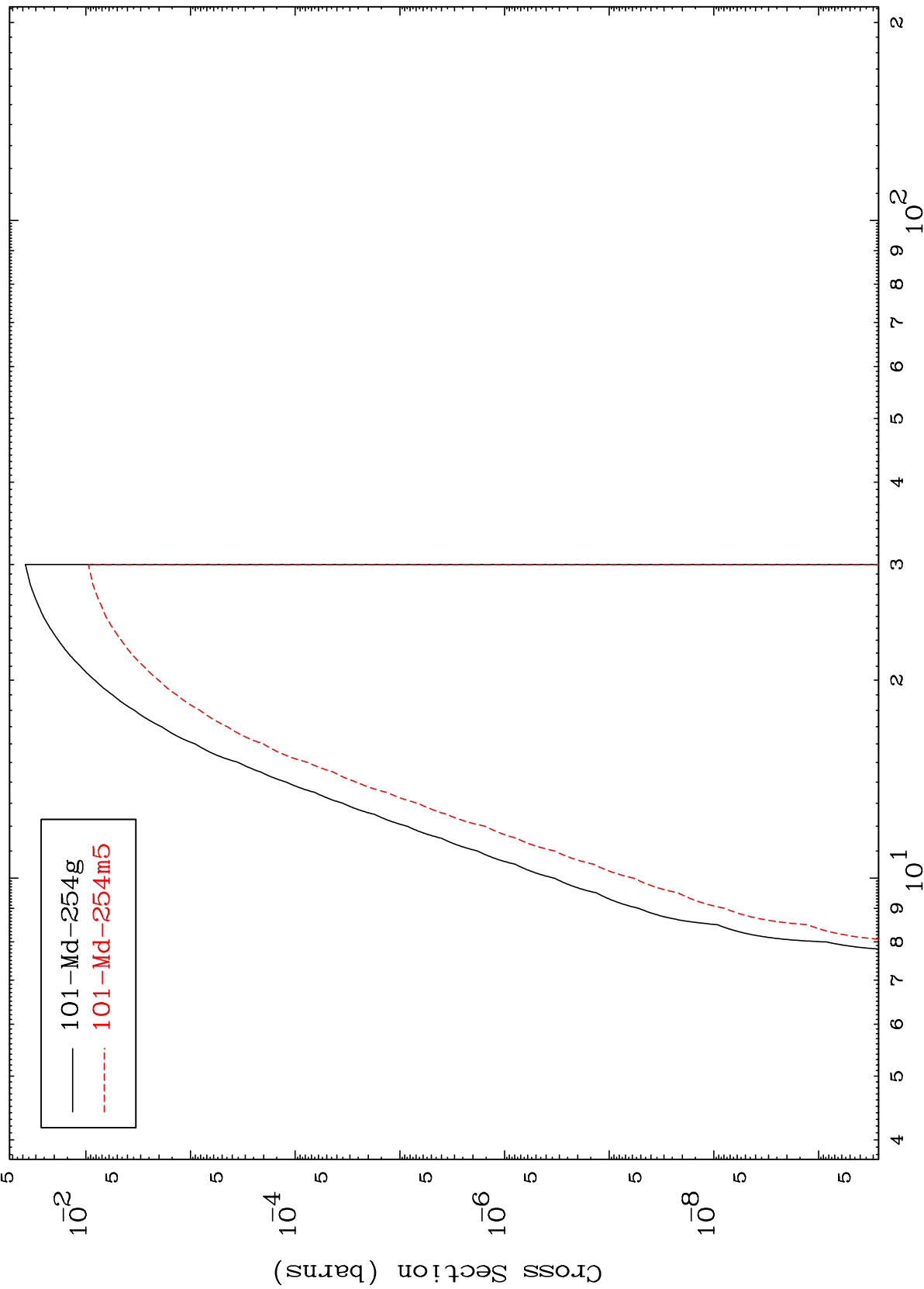


MAT 9973

(n,n') p

102-No-255

Radionuclide Production Cross Section



27

Incident Energy (MeV)

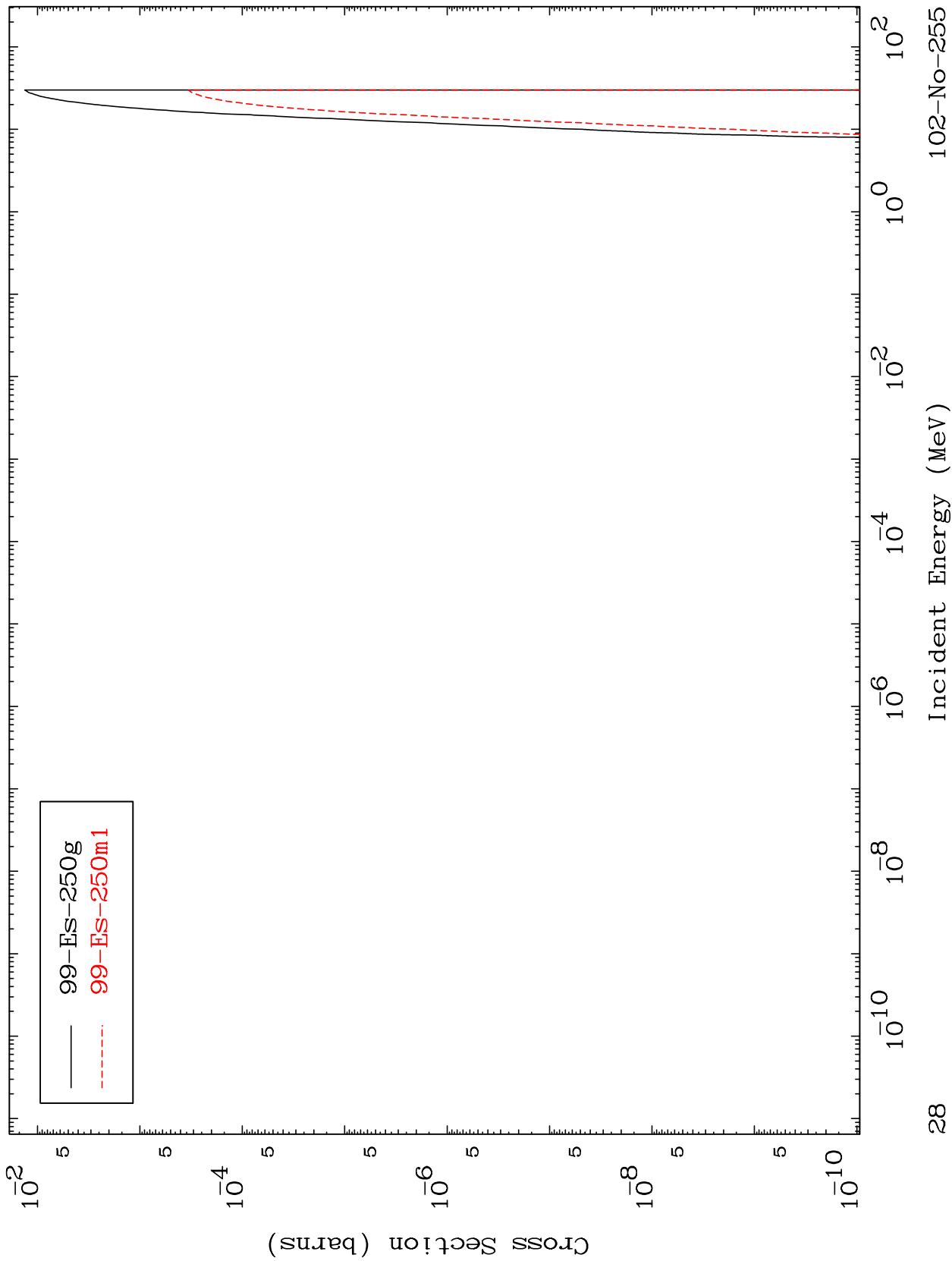
102-No-255

MAT 9973

(n,n') p α

102-No-255

Radionuclide Production Cross Section

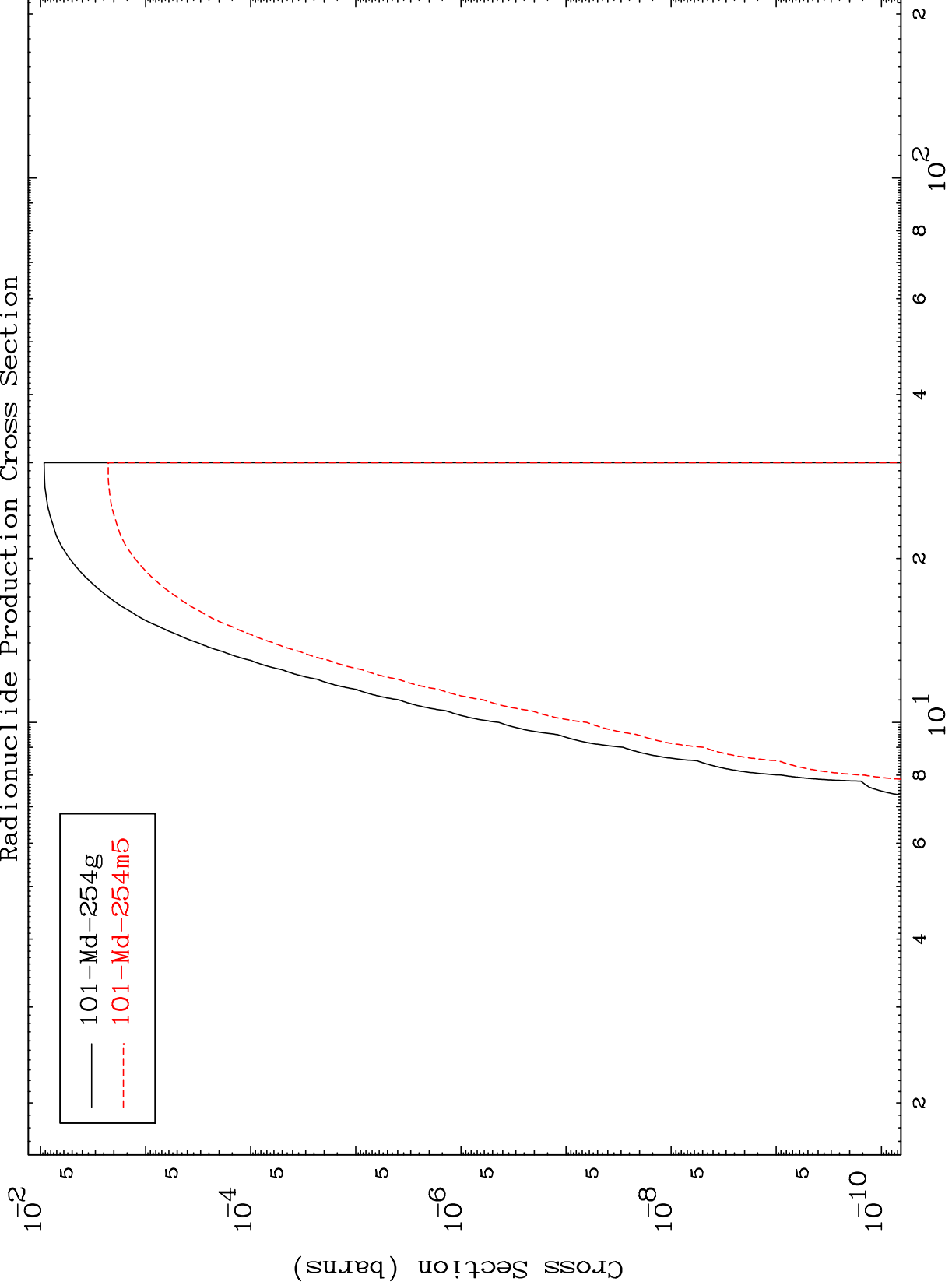


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(n,d)

102-No-255

Radionuclide Production Cross Section



101-Md-254g
101-Md-254m5

MAT 9973

(n,d) α

102-No-255

Radionuclide Production Cross Section

