

Program EVALPLOT
(Version 2015-2)

by

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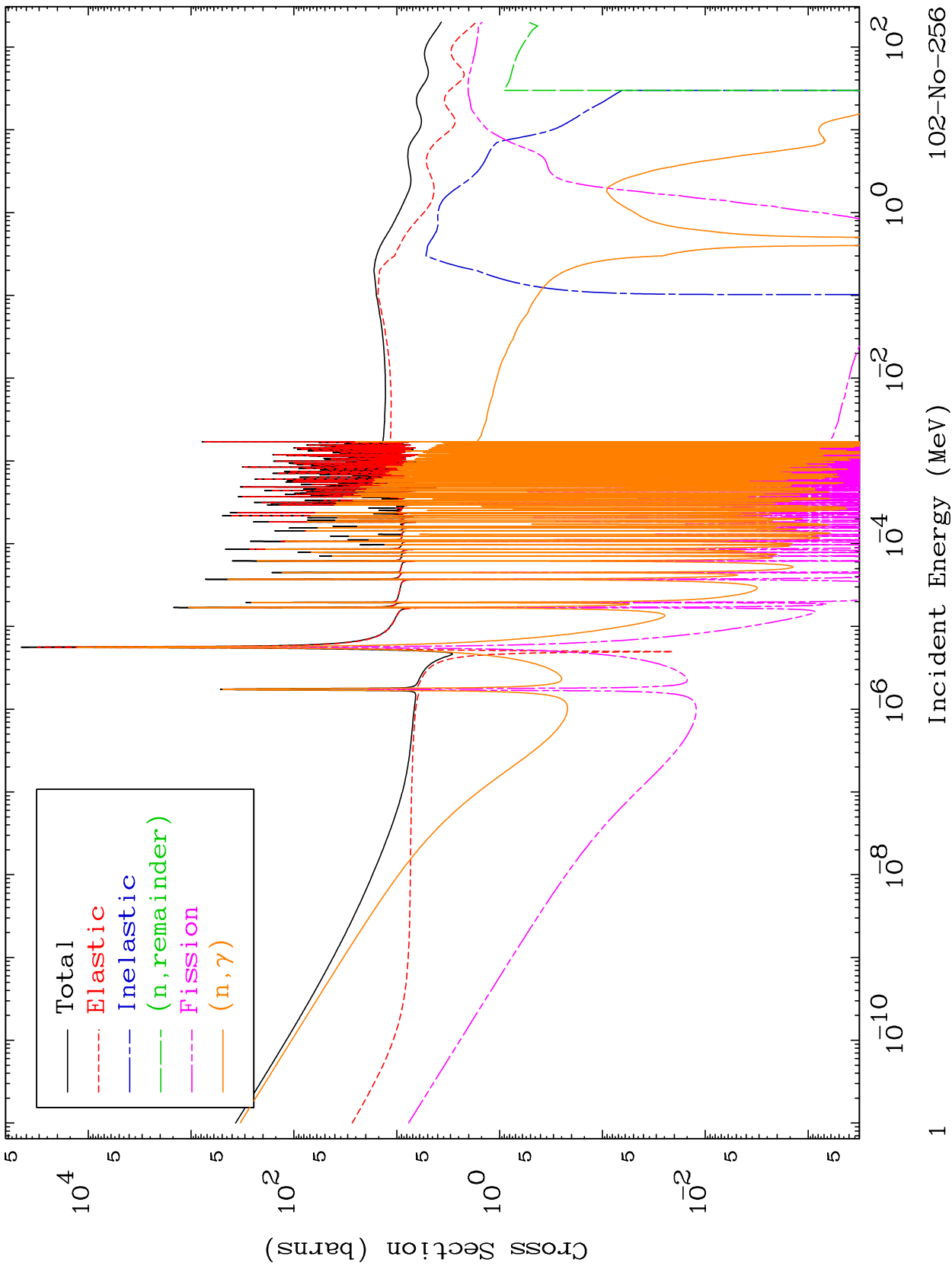
Press Mouse Button to Start

MAT 9974

Major

102-No-256

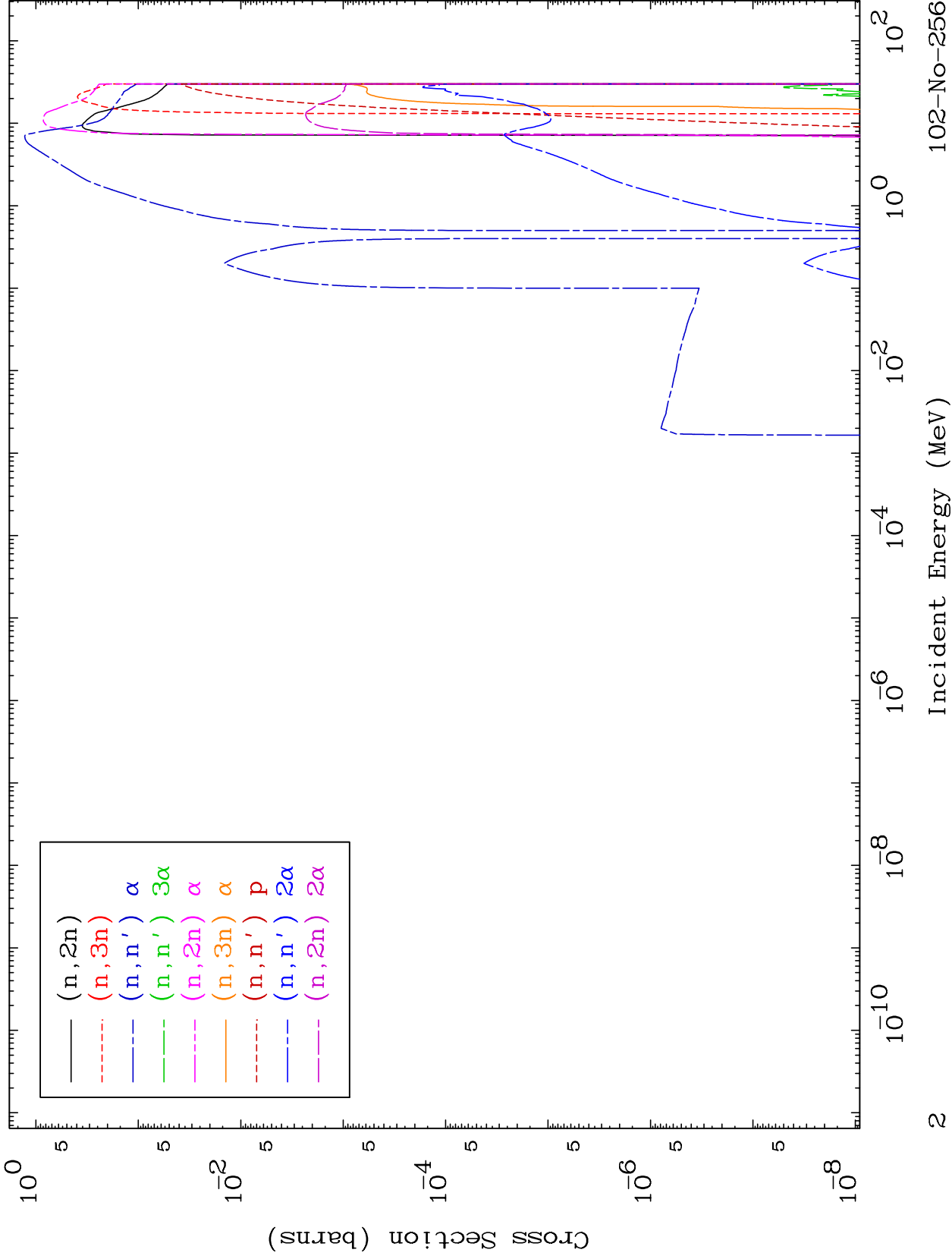
293 Kelvin Cross Sections



MAT 9974

Neutron Production
293 Kelvin Cross Sections

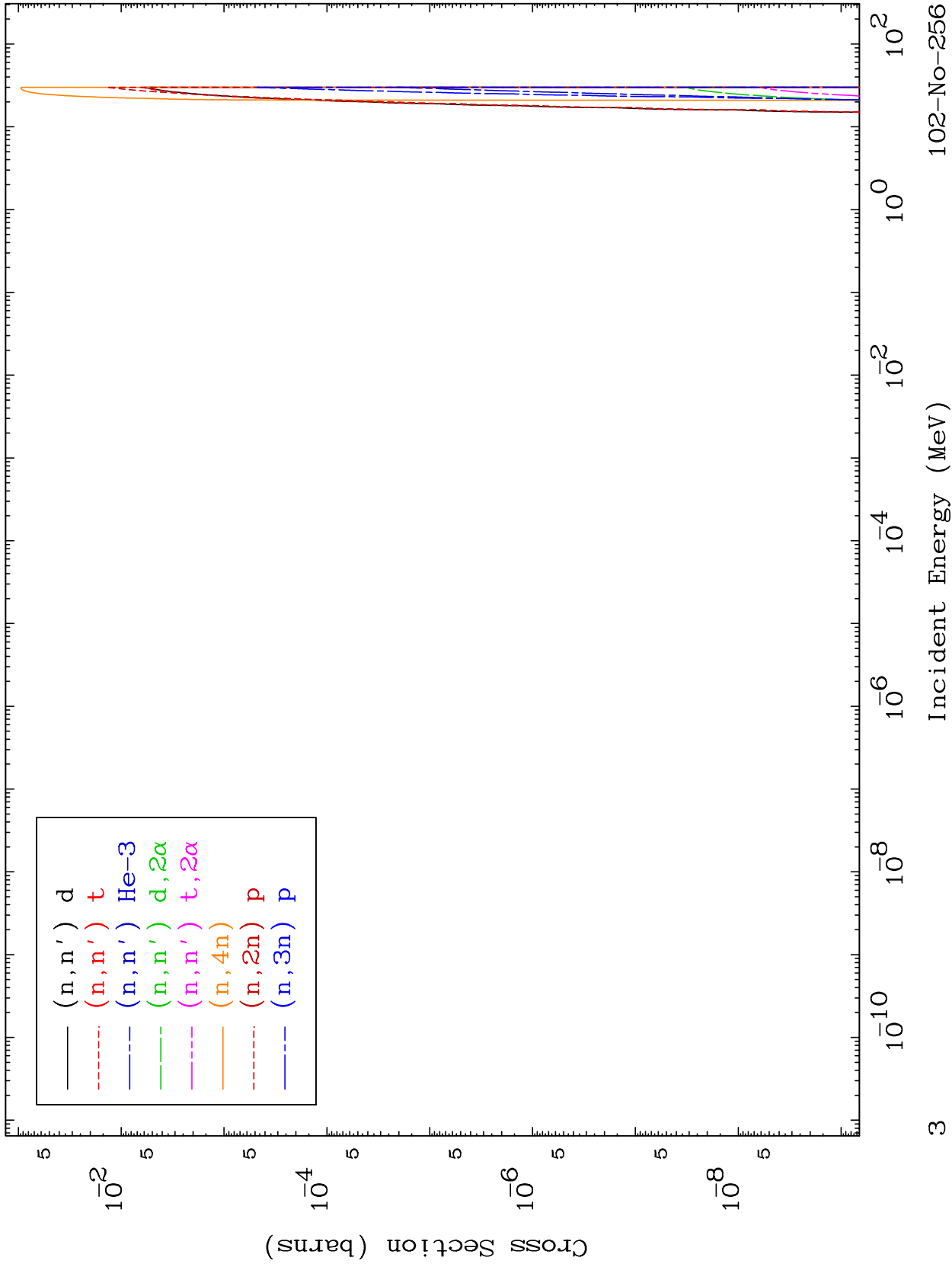
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MAT 9974

Neutron Production
293 Kelvin Cross Sections

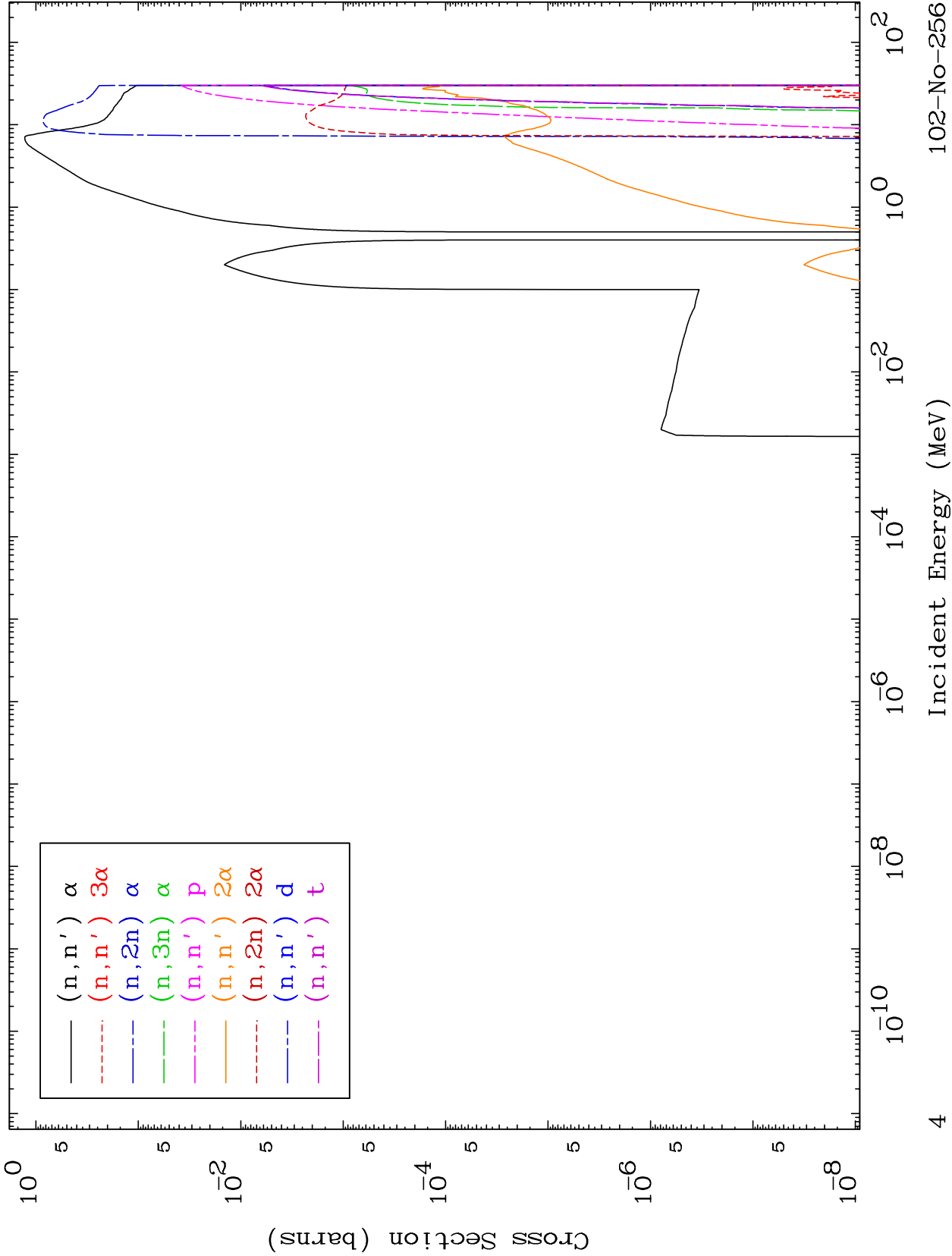
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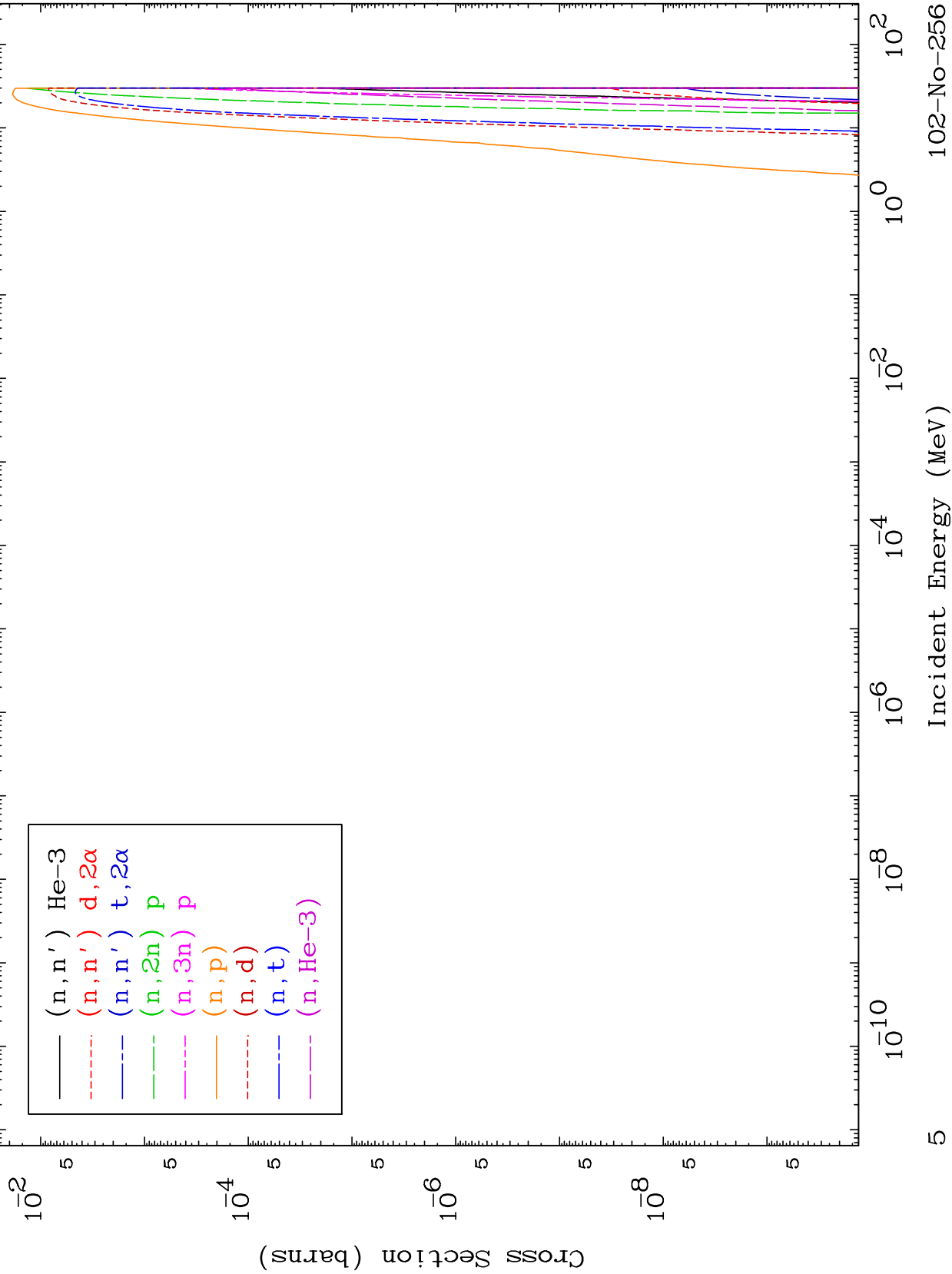


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Charged Particle
293 Kelvin Cross Sections

102-No-256

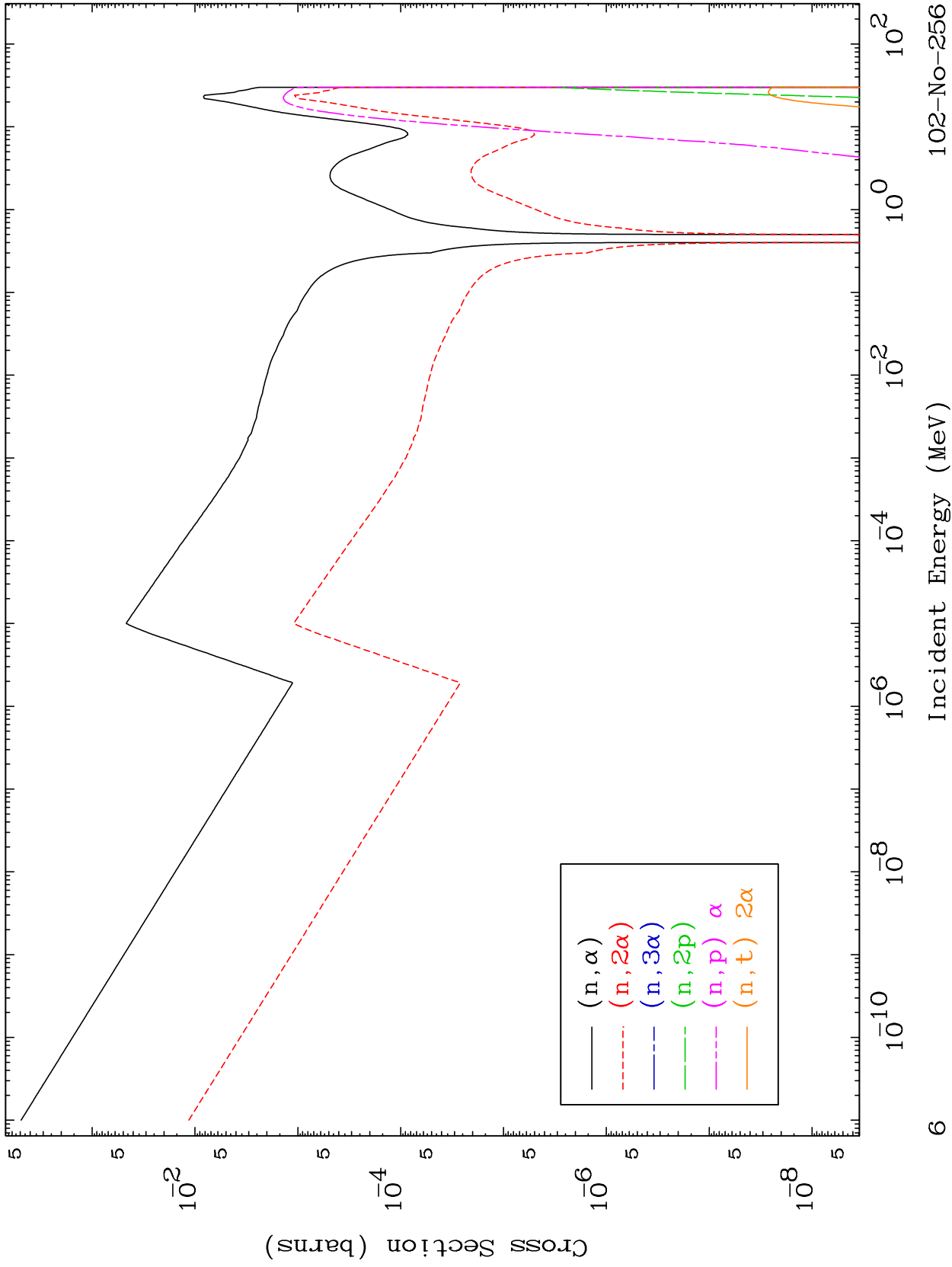




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Charged Particle
293 Kelvin Cross Sections

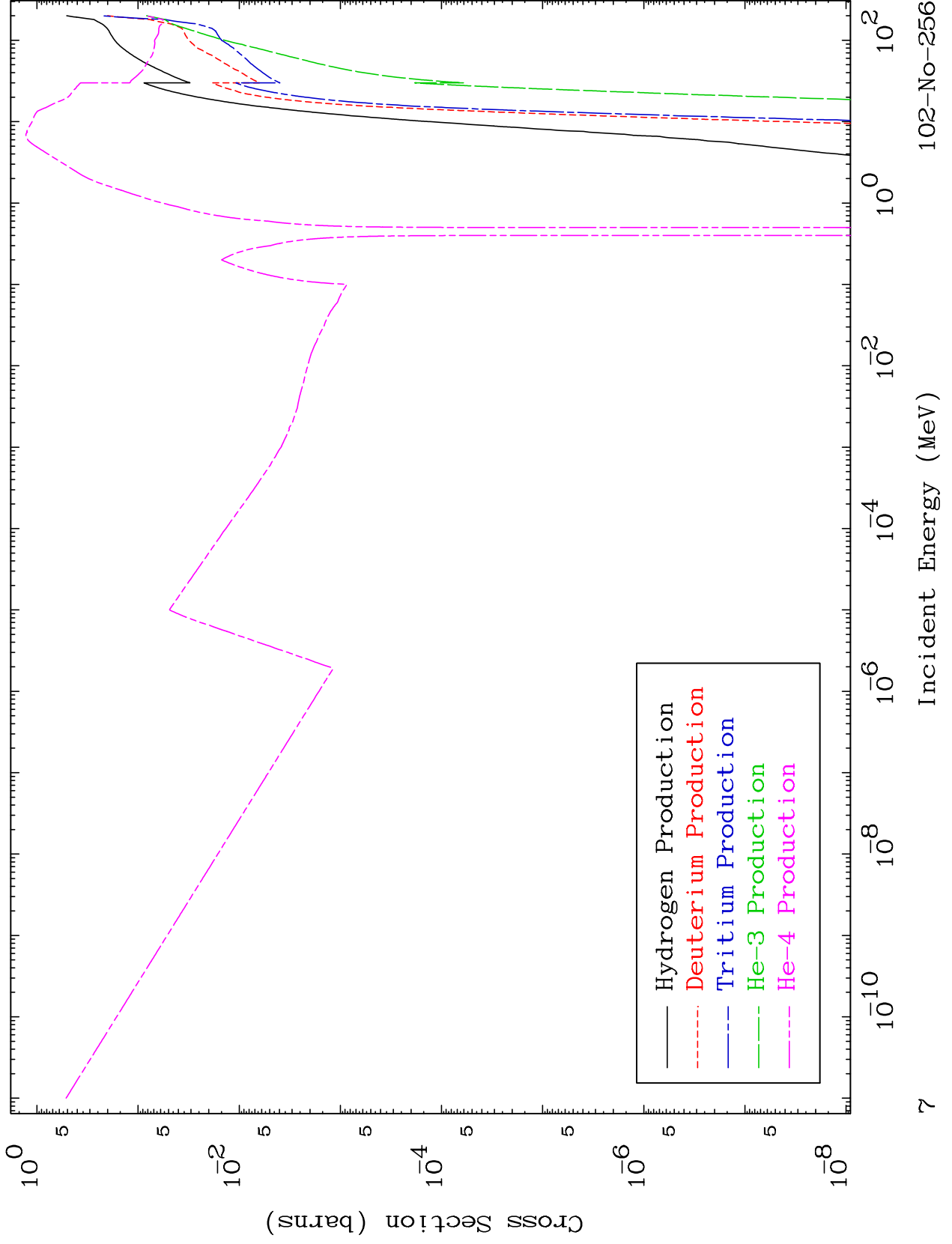
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Particle Production
293 Kelvin Cross Sections

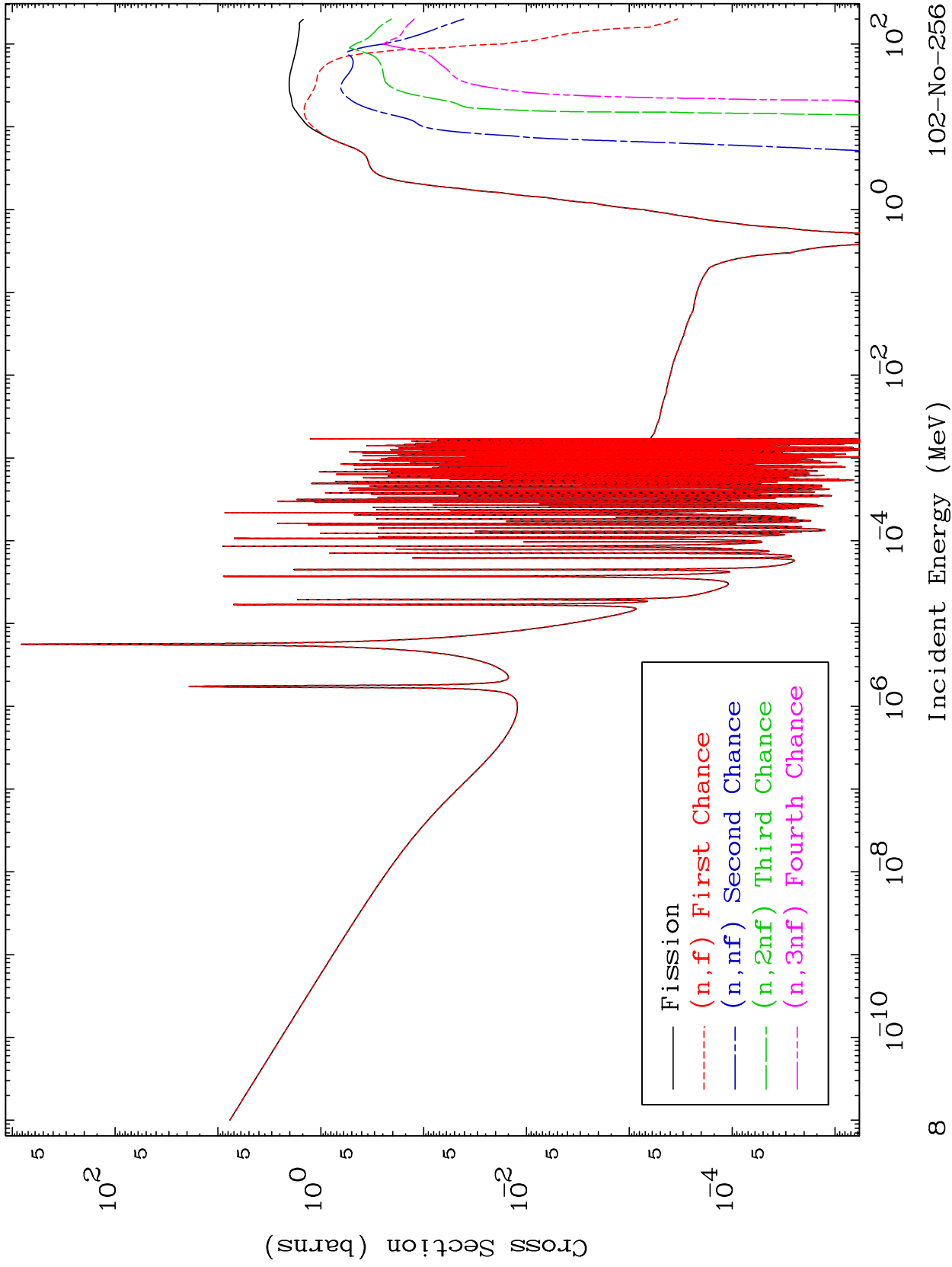
102-No-256



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Fission
293 Kelvin Cross Sections

102-No-256

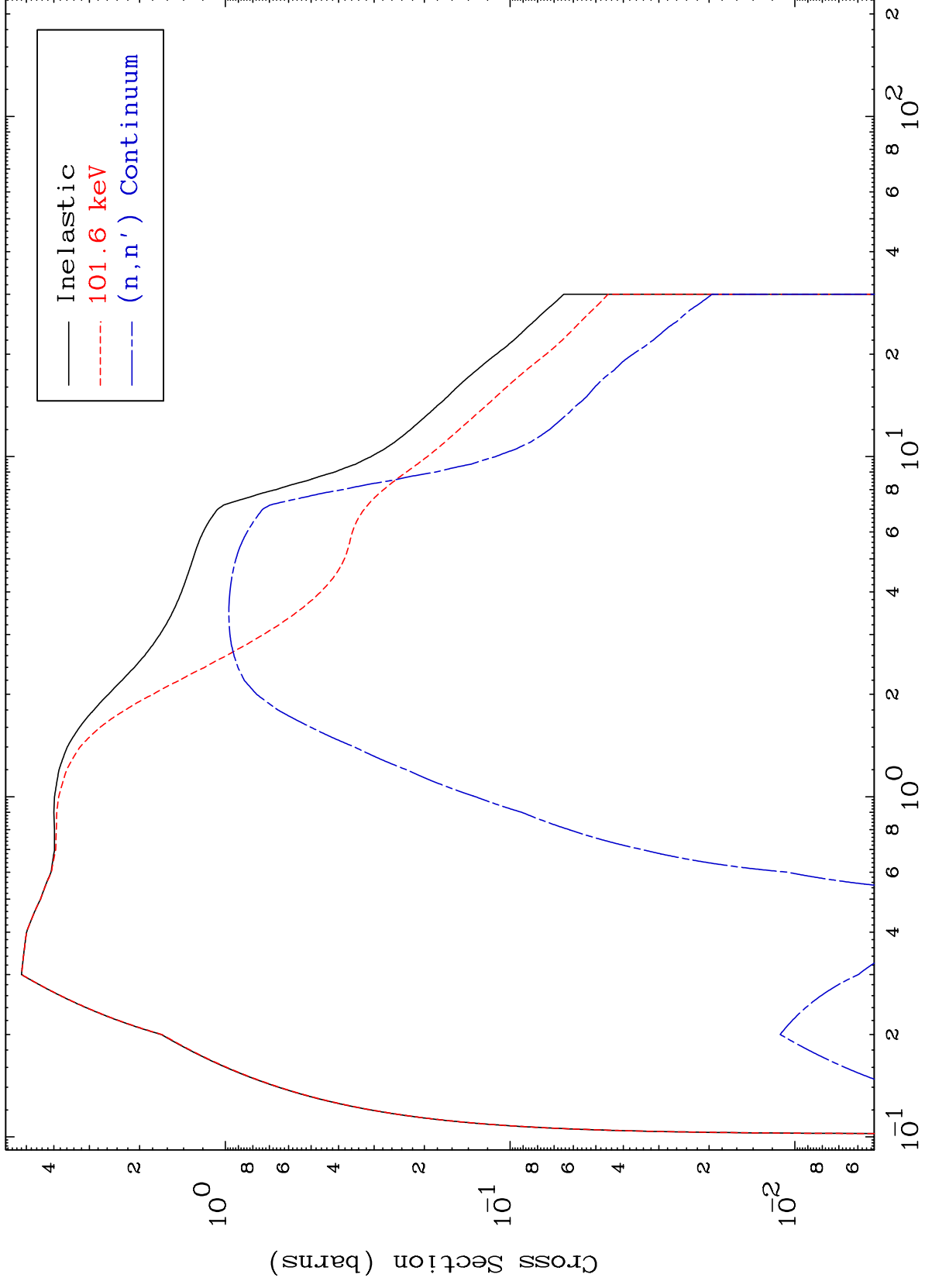


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(n,n') Level

102-No-256

293 Kelvin Cross Sections



Incident Energy (MeV)

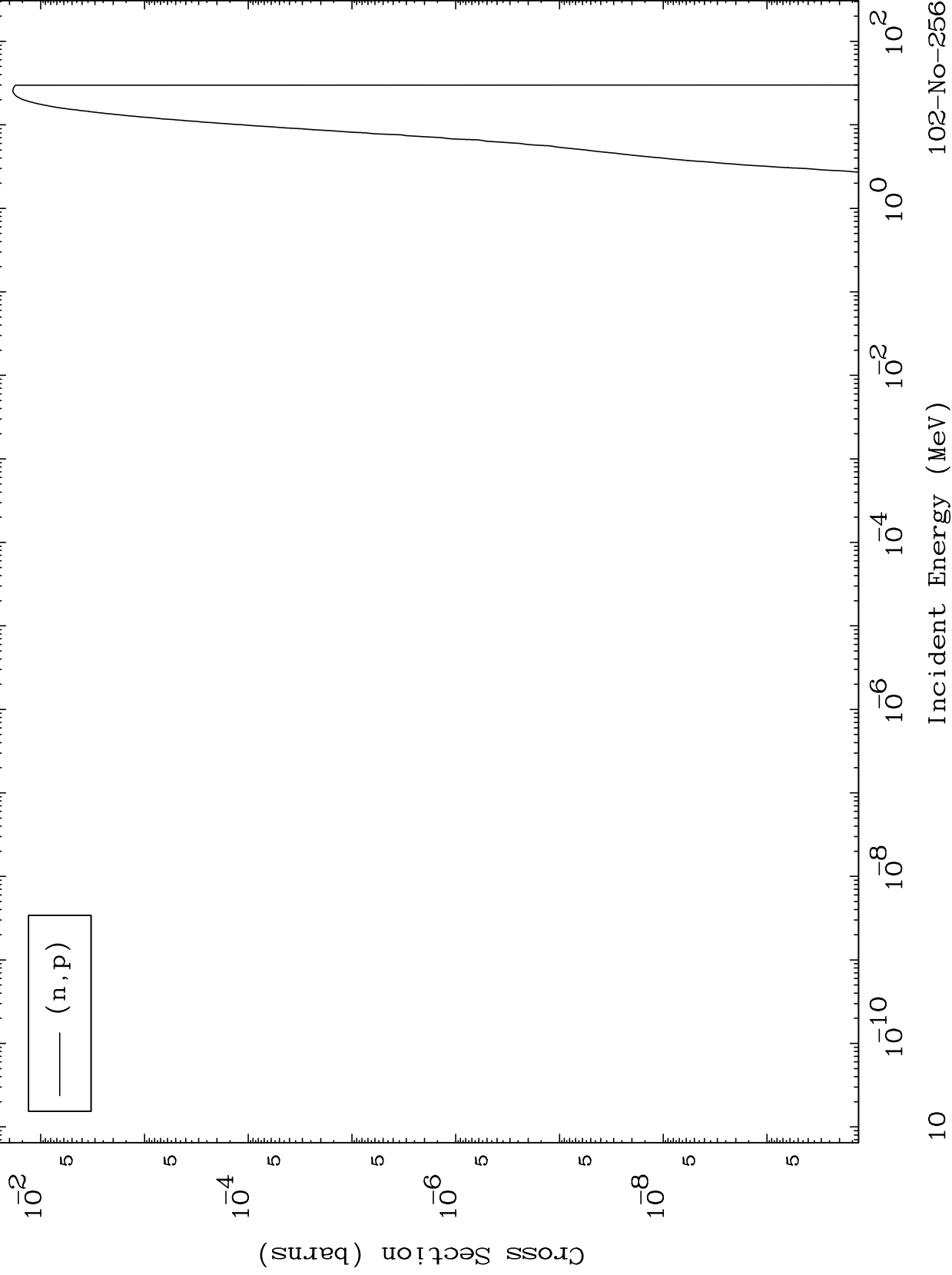
102-No-256

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(n,p) Levels
293 Kelvin Cross Sections

102-No-256



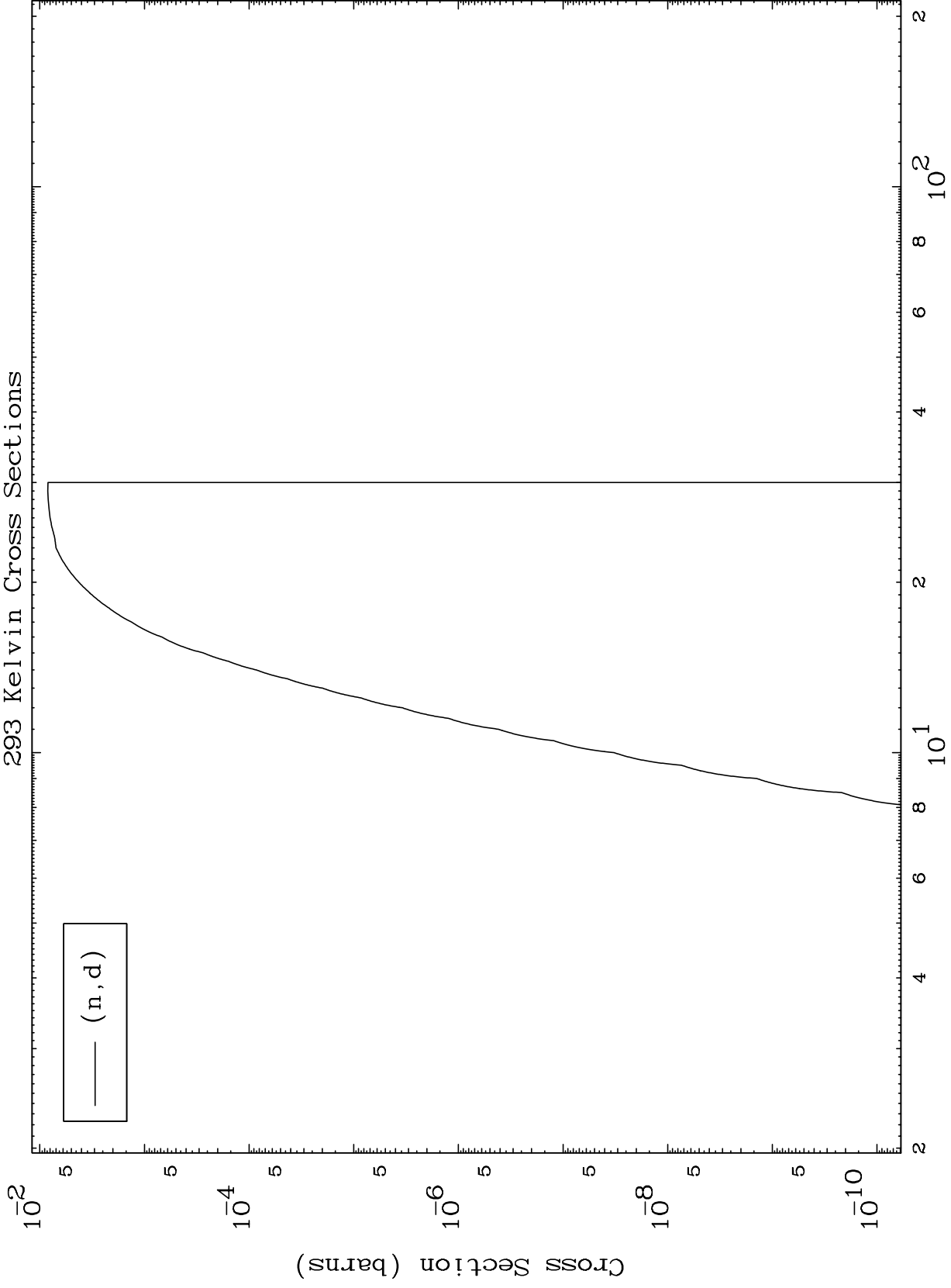
10

102-No-256

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(n,d) Levels
293 Kelvin Cross Sections

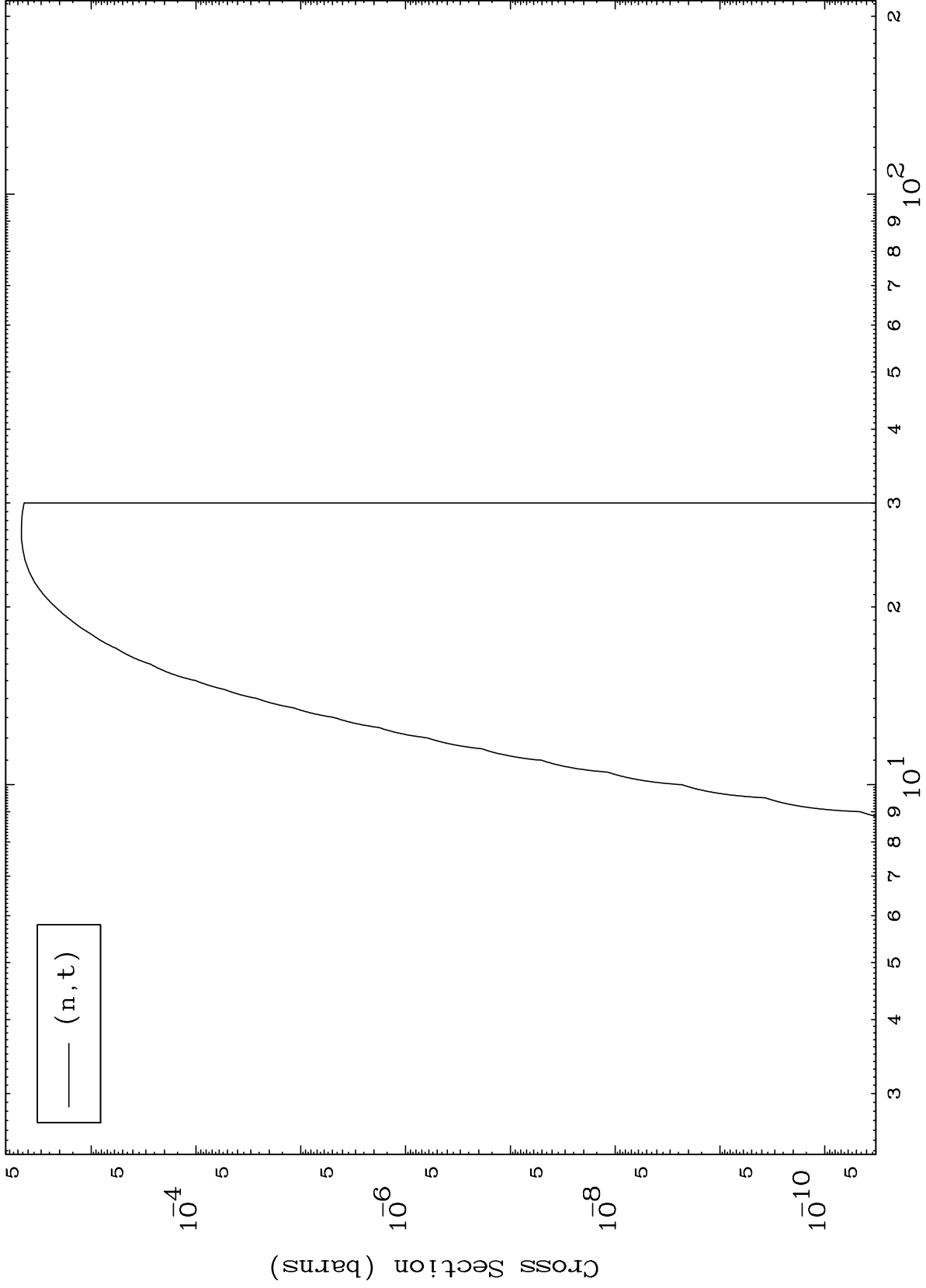
102-No-256



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(n,t) Levels
293 Kelvin Cross Sections

102-No-256



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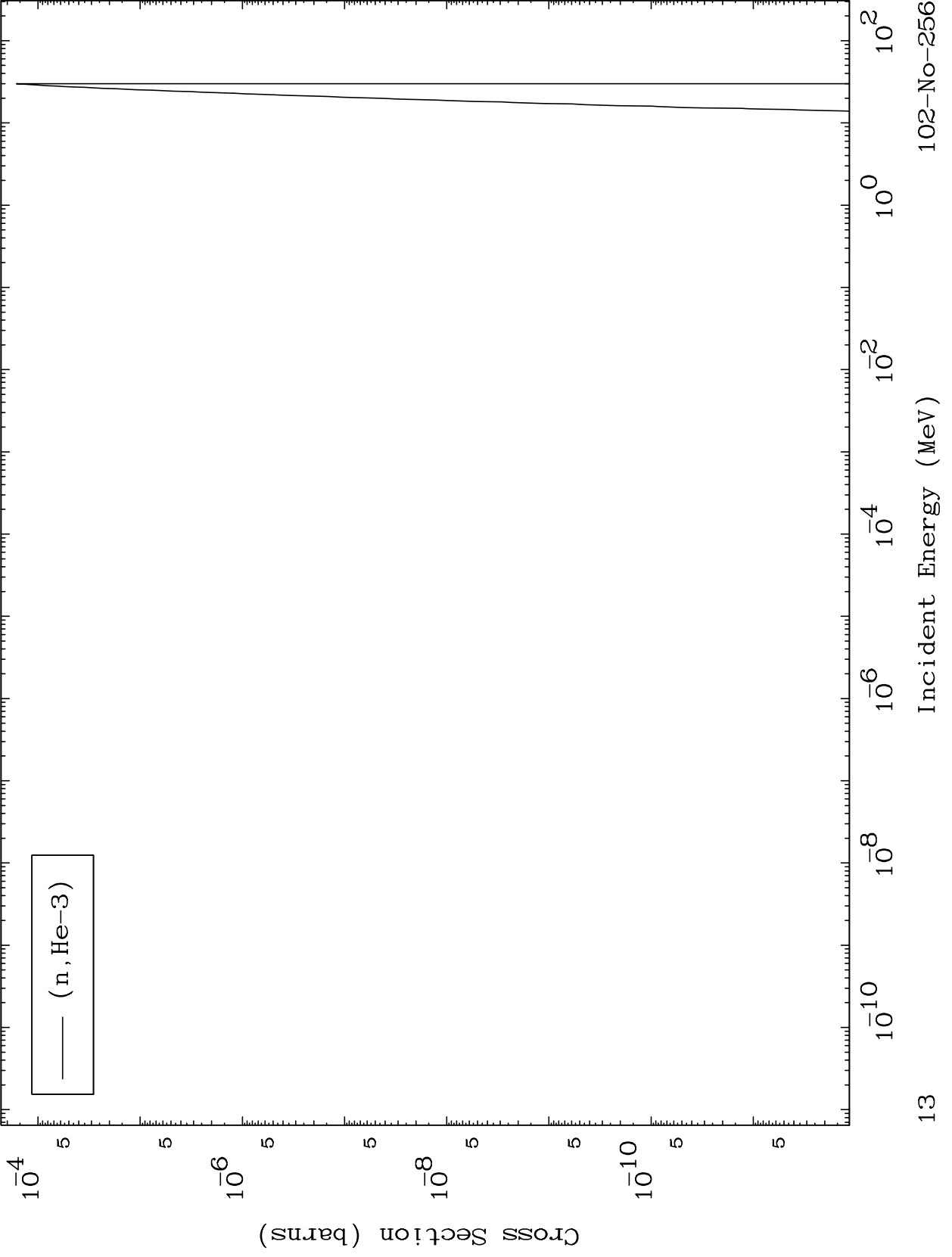
Incident Energy (MeV)

102-No-256

MAT 9974

(n,He3) Levels
293 Kelvin Cross Sections

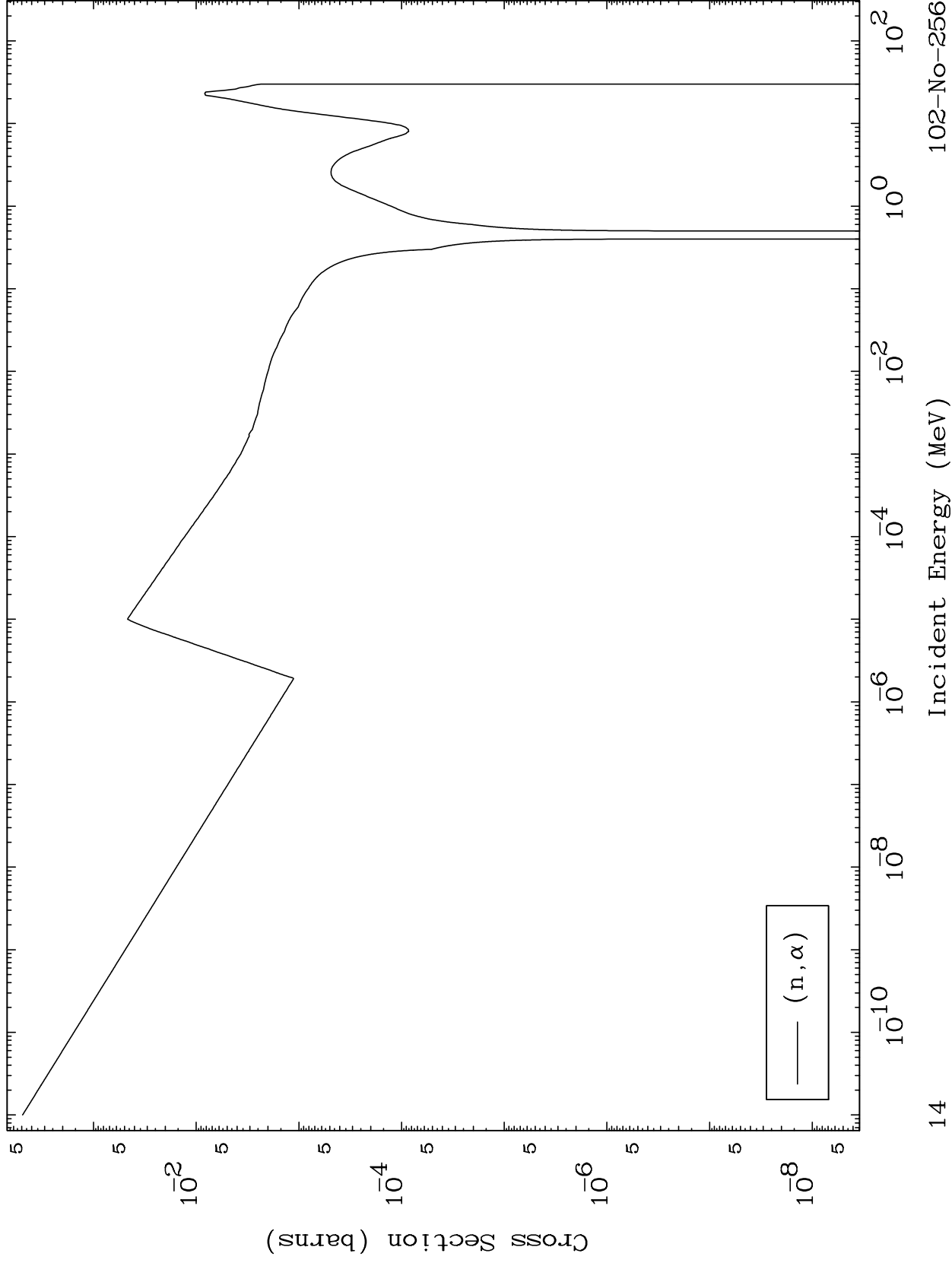
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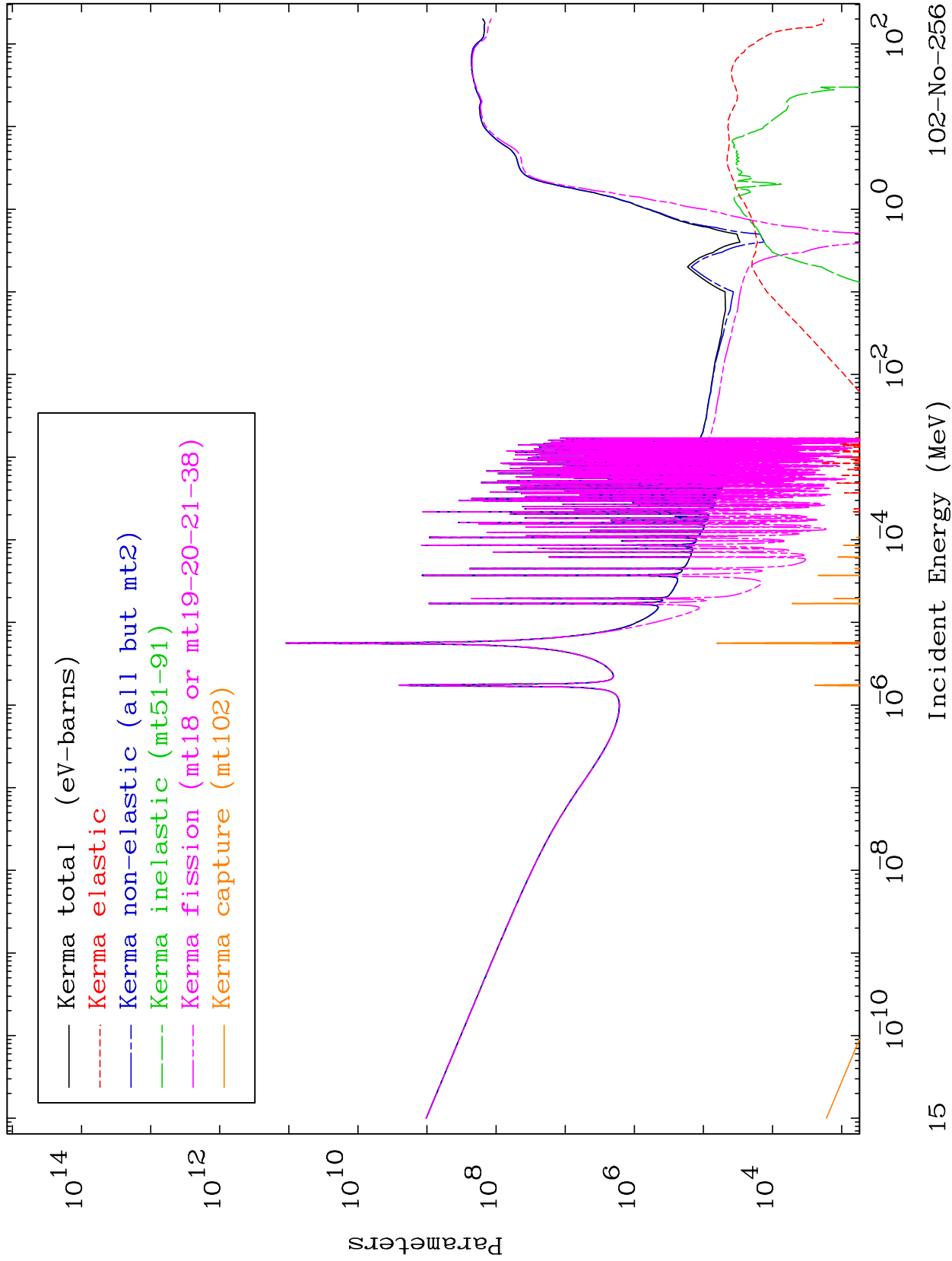


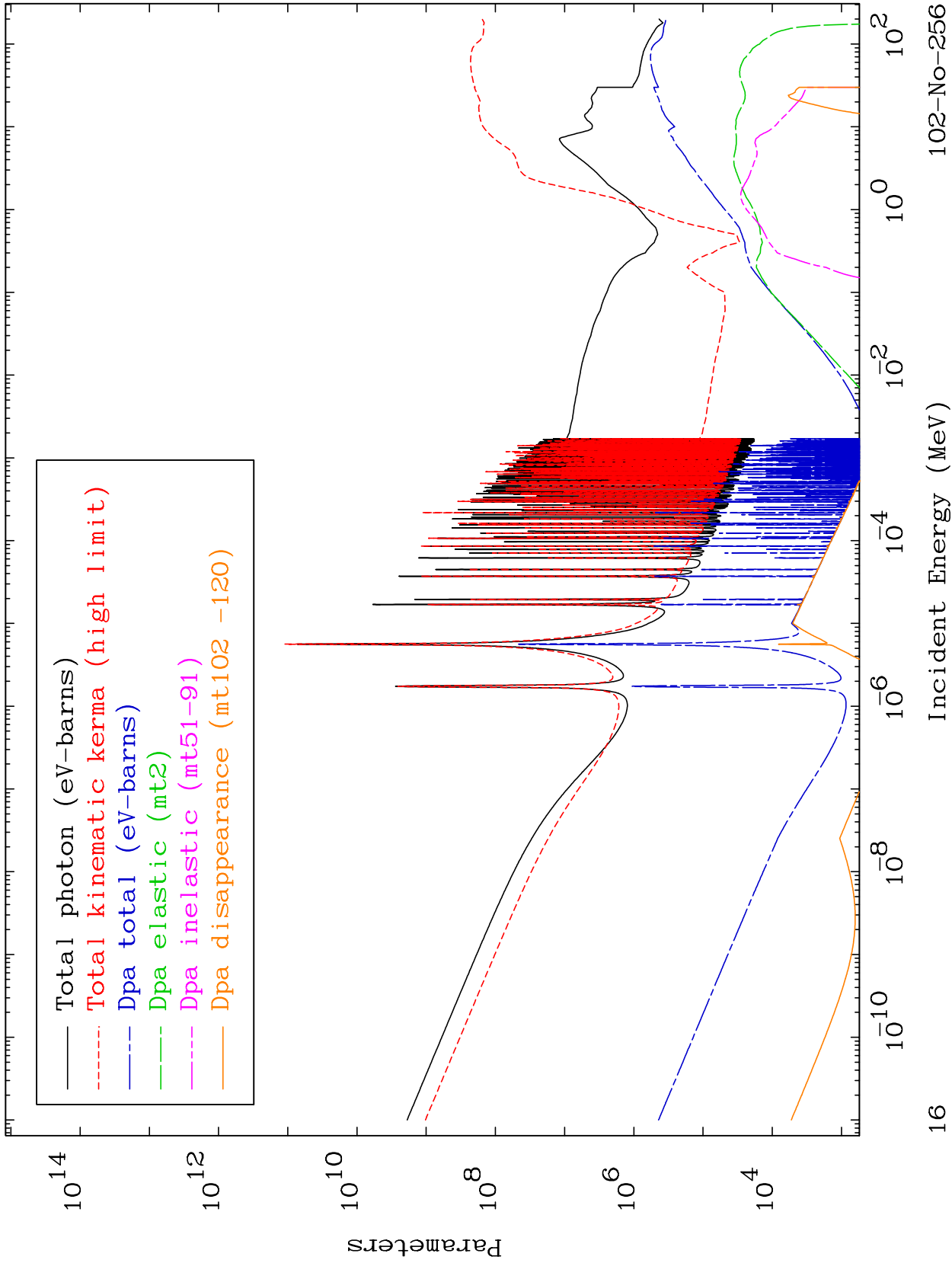
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(n,α) Levels
293 Kelvin Cross Sections

102-No-256



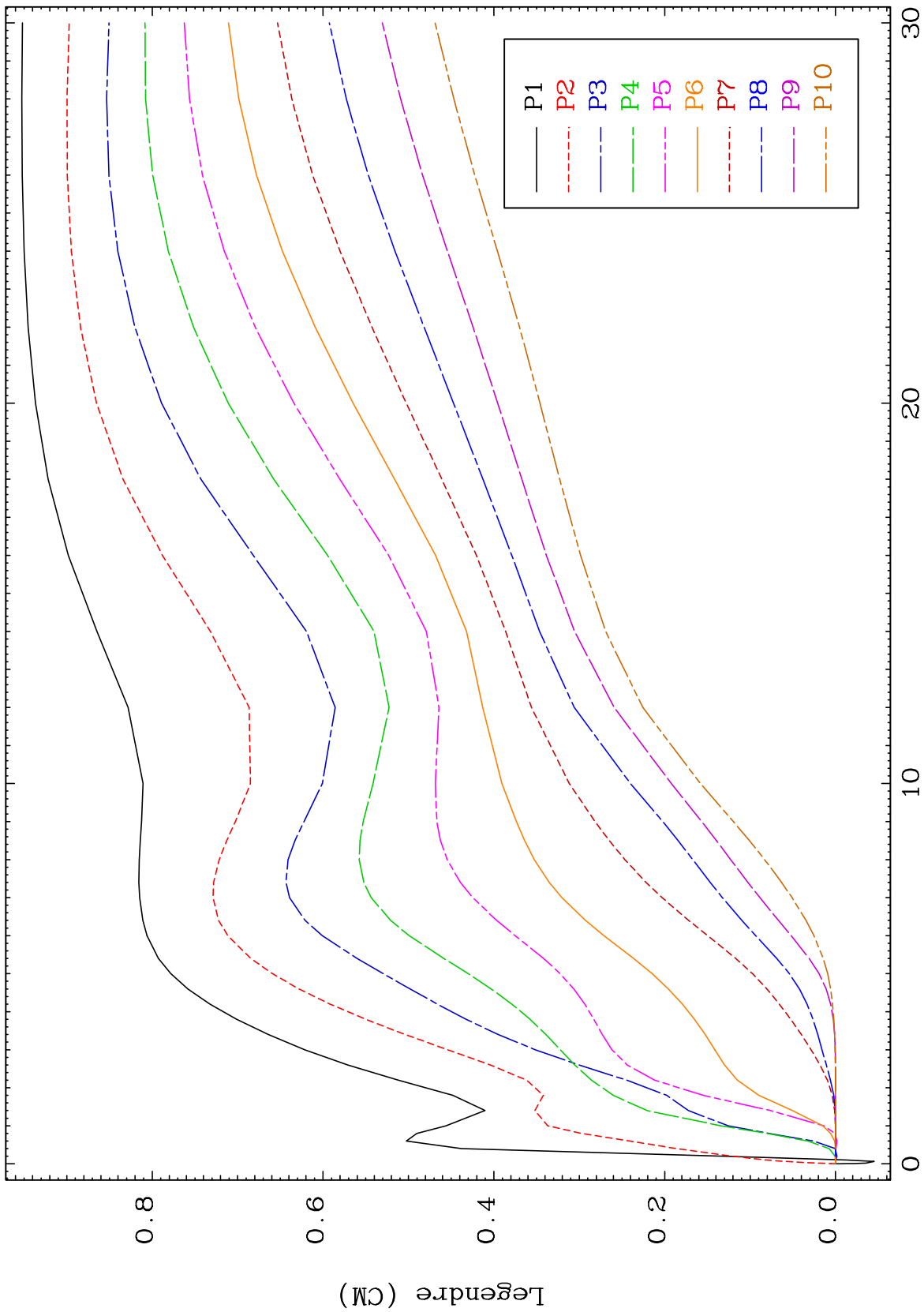




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Elastic Legendre Coefficients

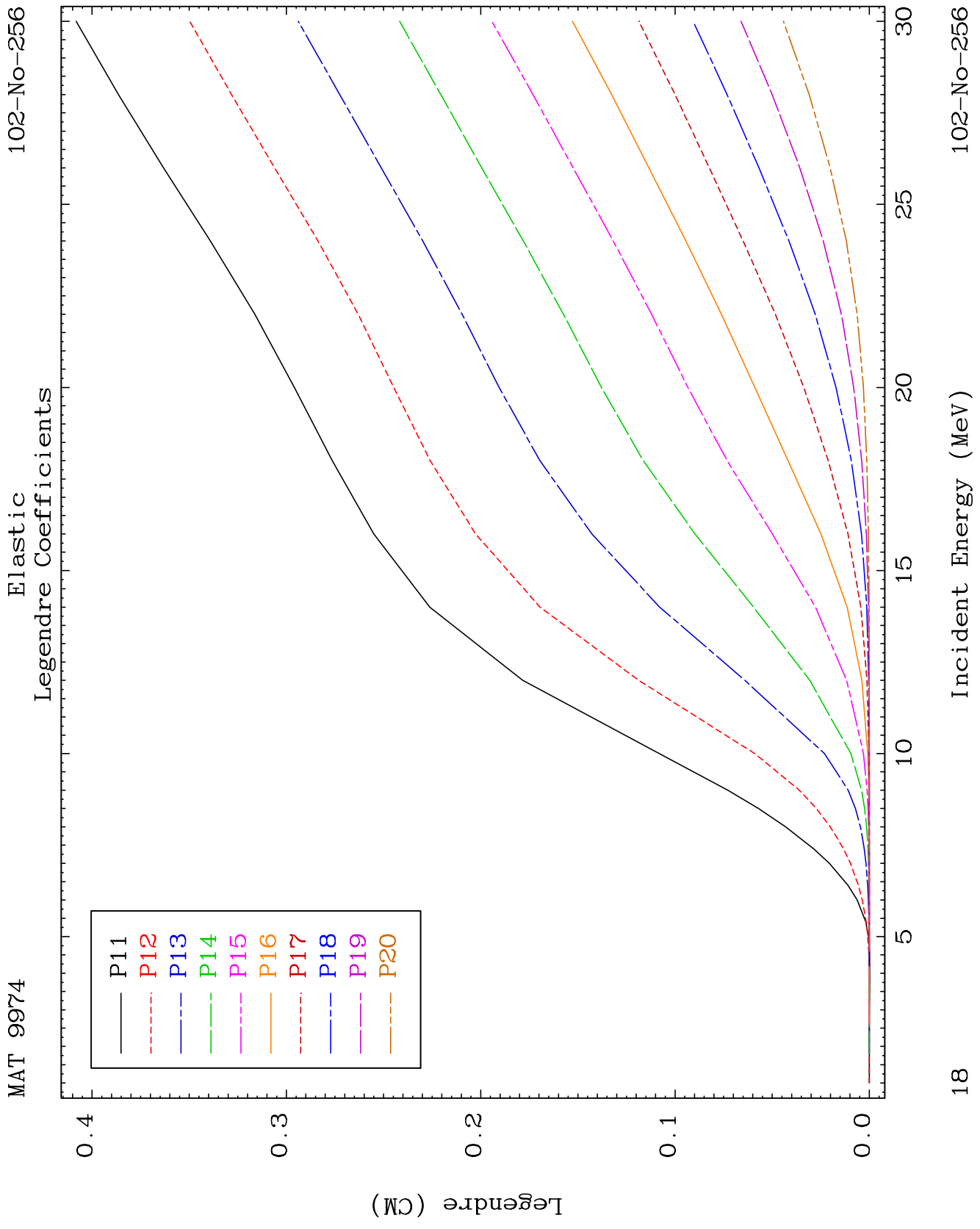
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Incident Energy (MeV)

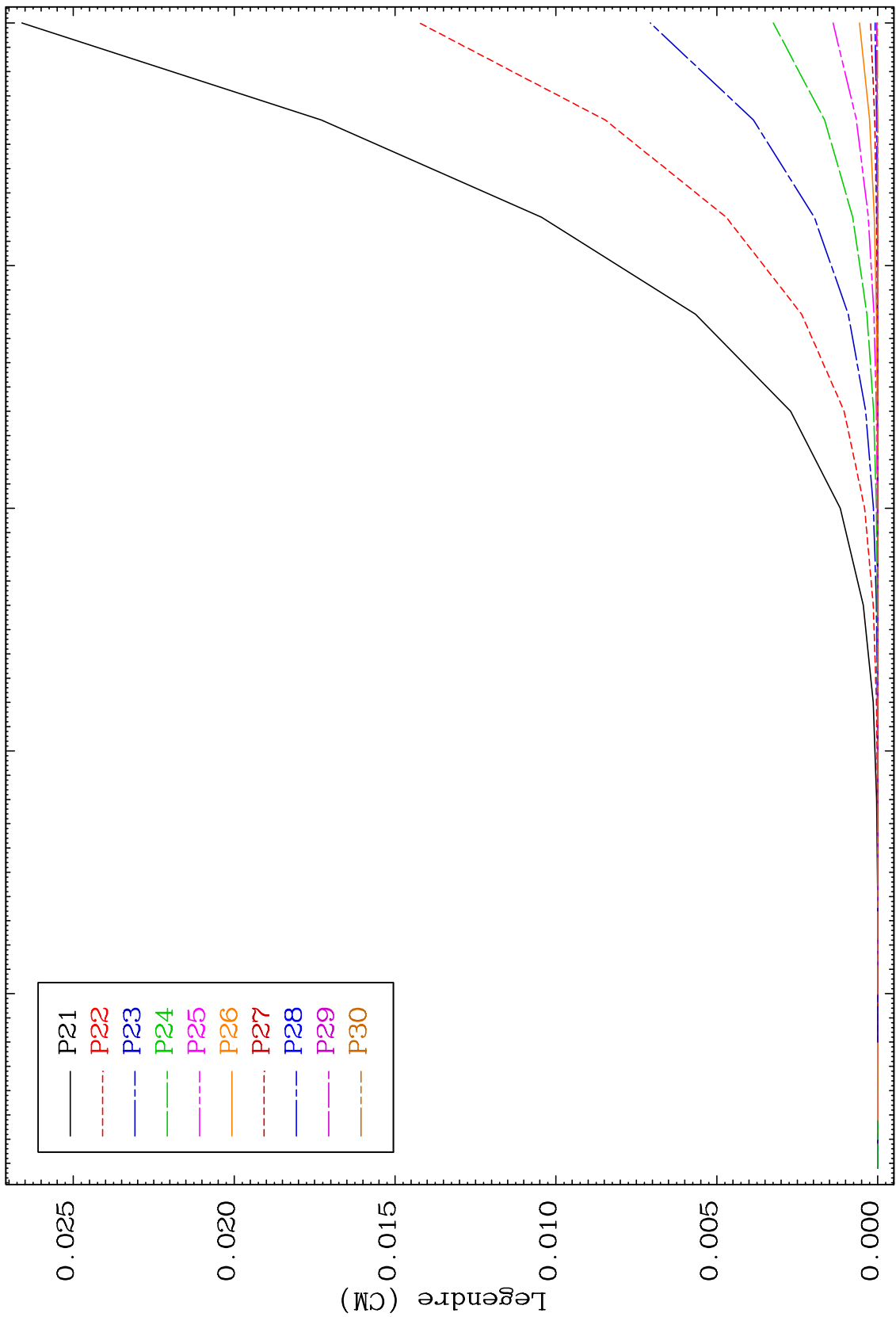
102-No-256



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Elastic Legendre Coefficients

102-No-256



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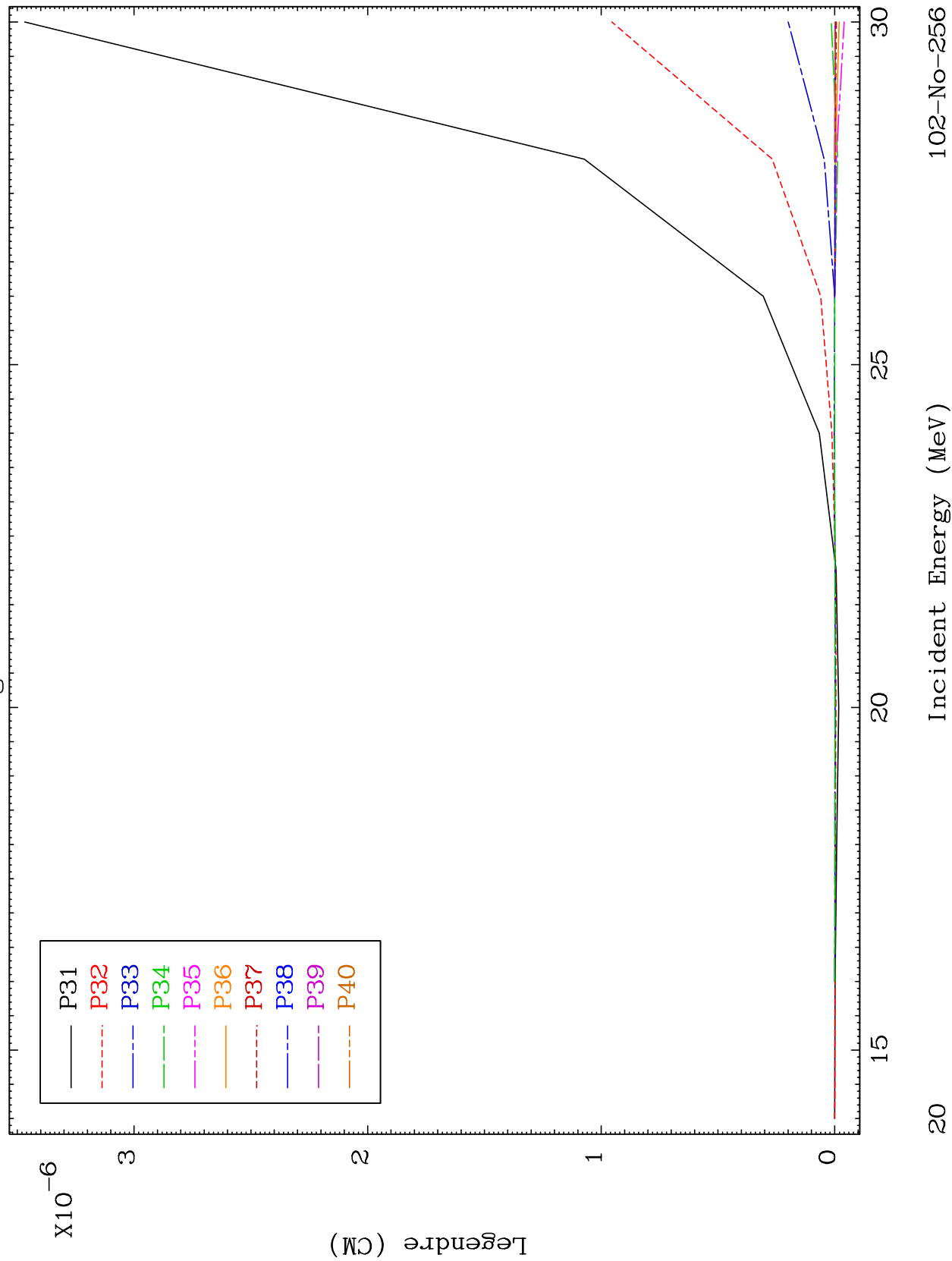
Incident Energy (MeV)

102-No-256

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Elastic Legendre Coefficients

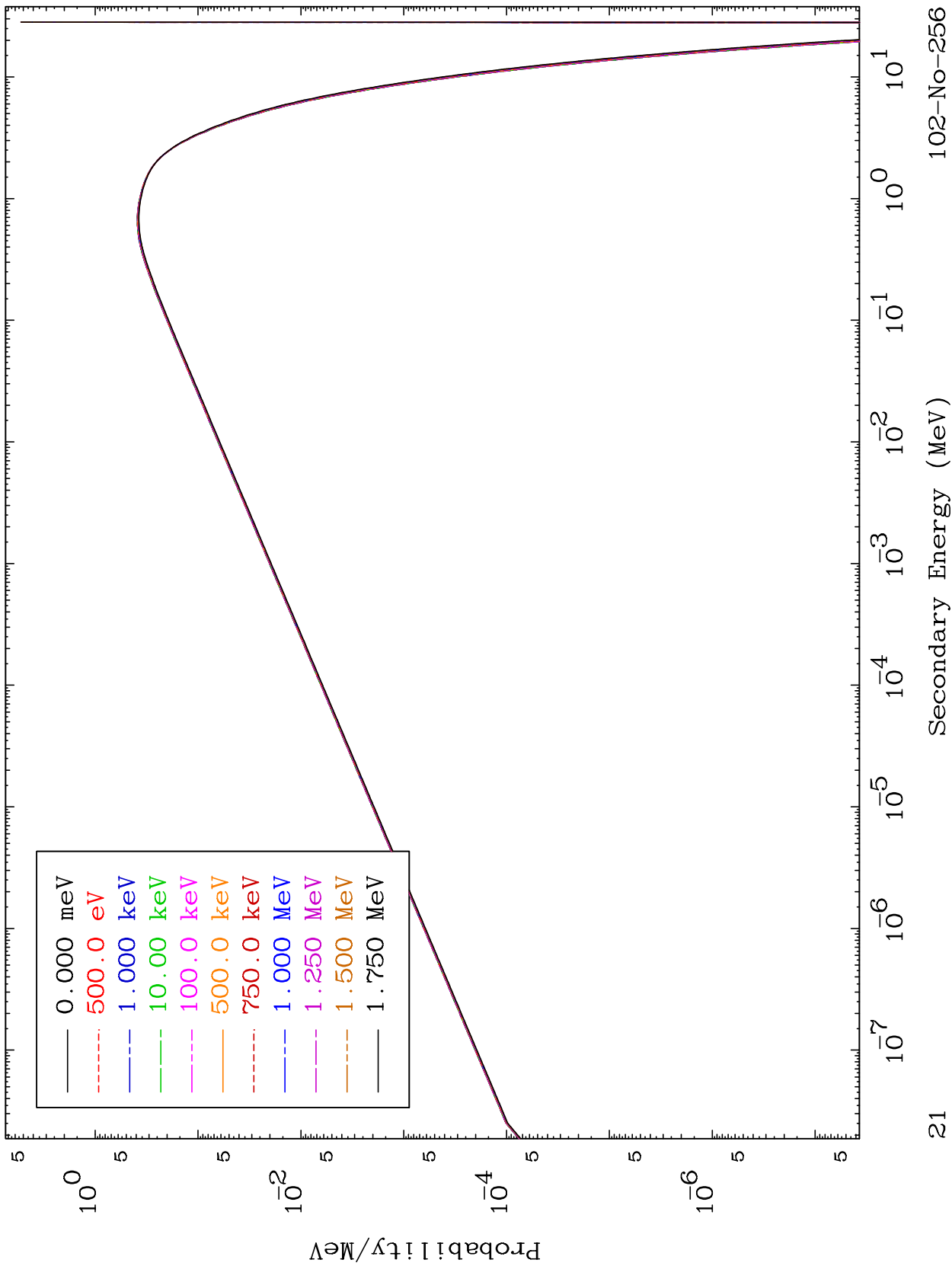
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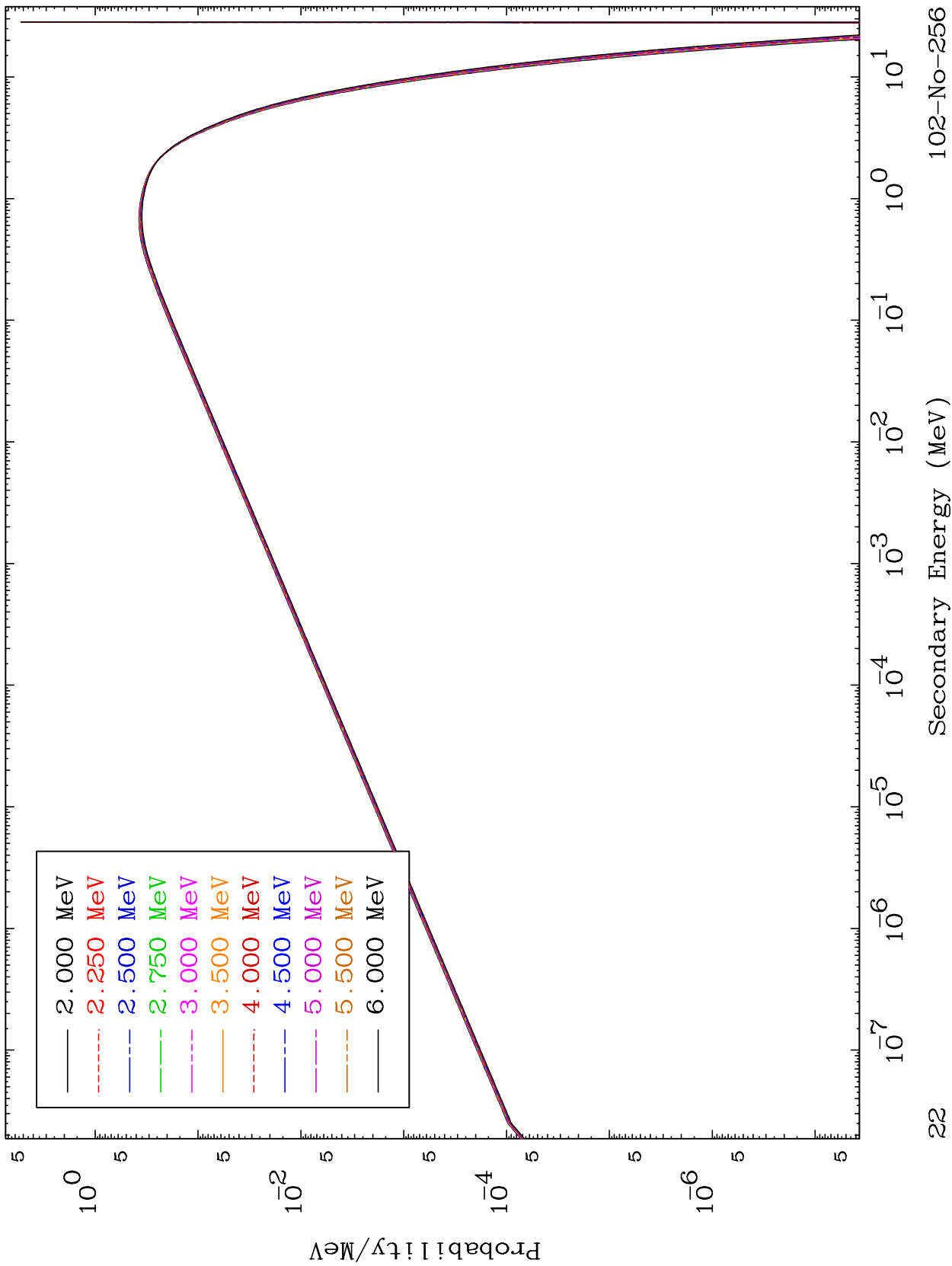


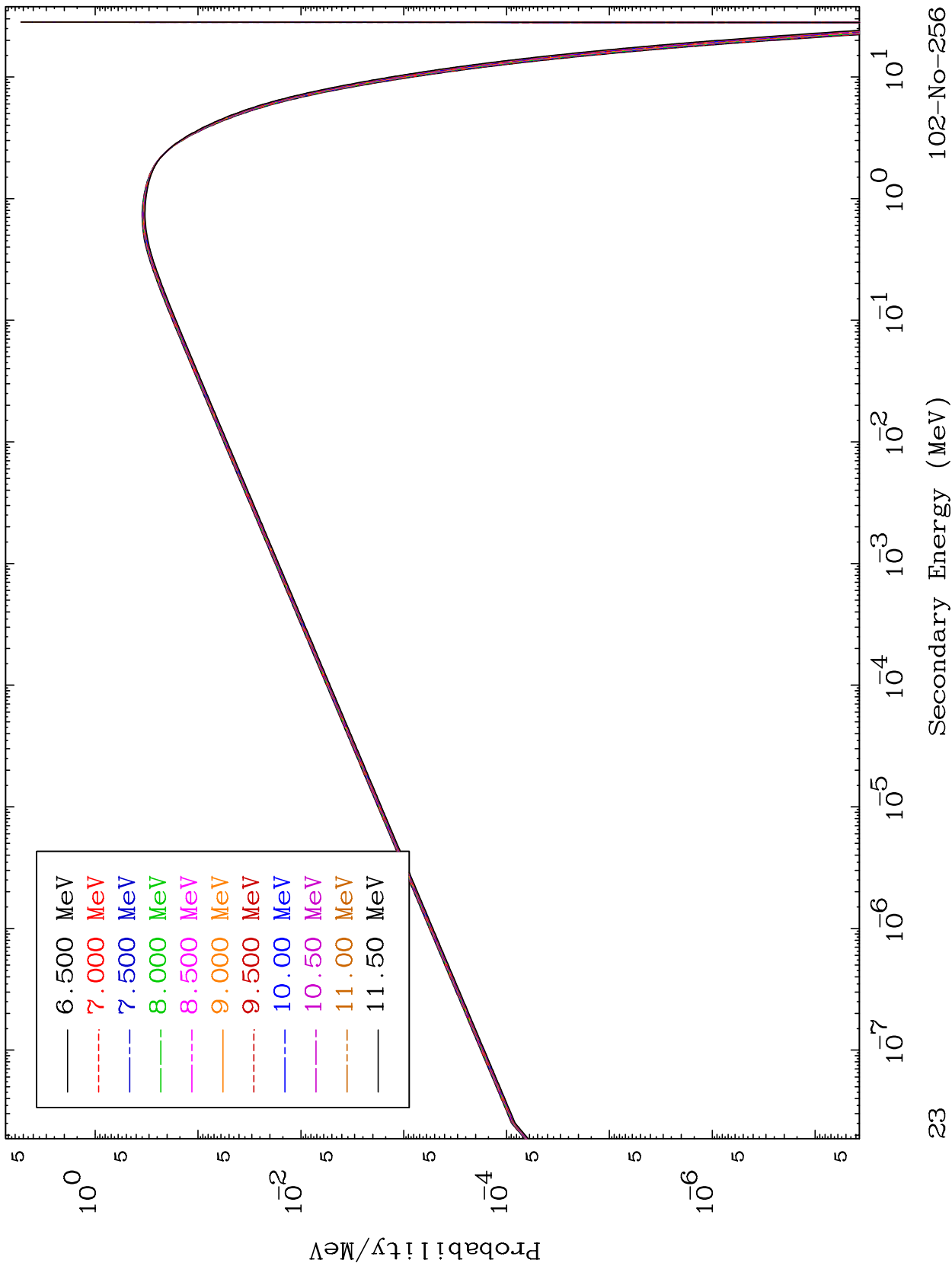
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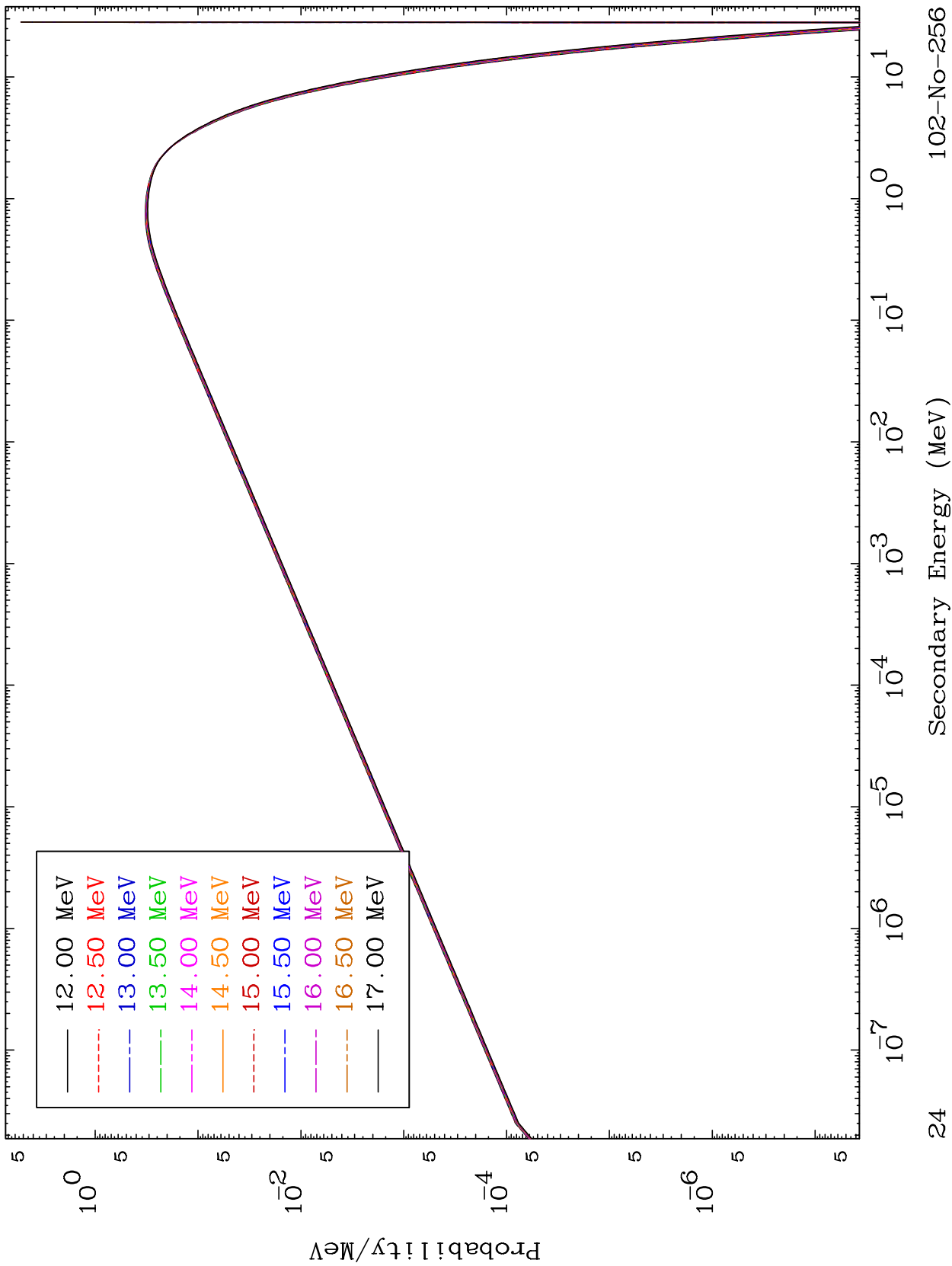
Incident Energy (MeV)

102-No-256

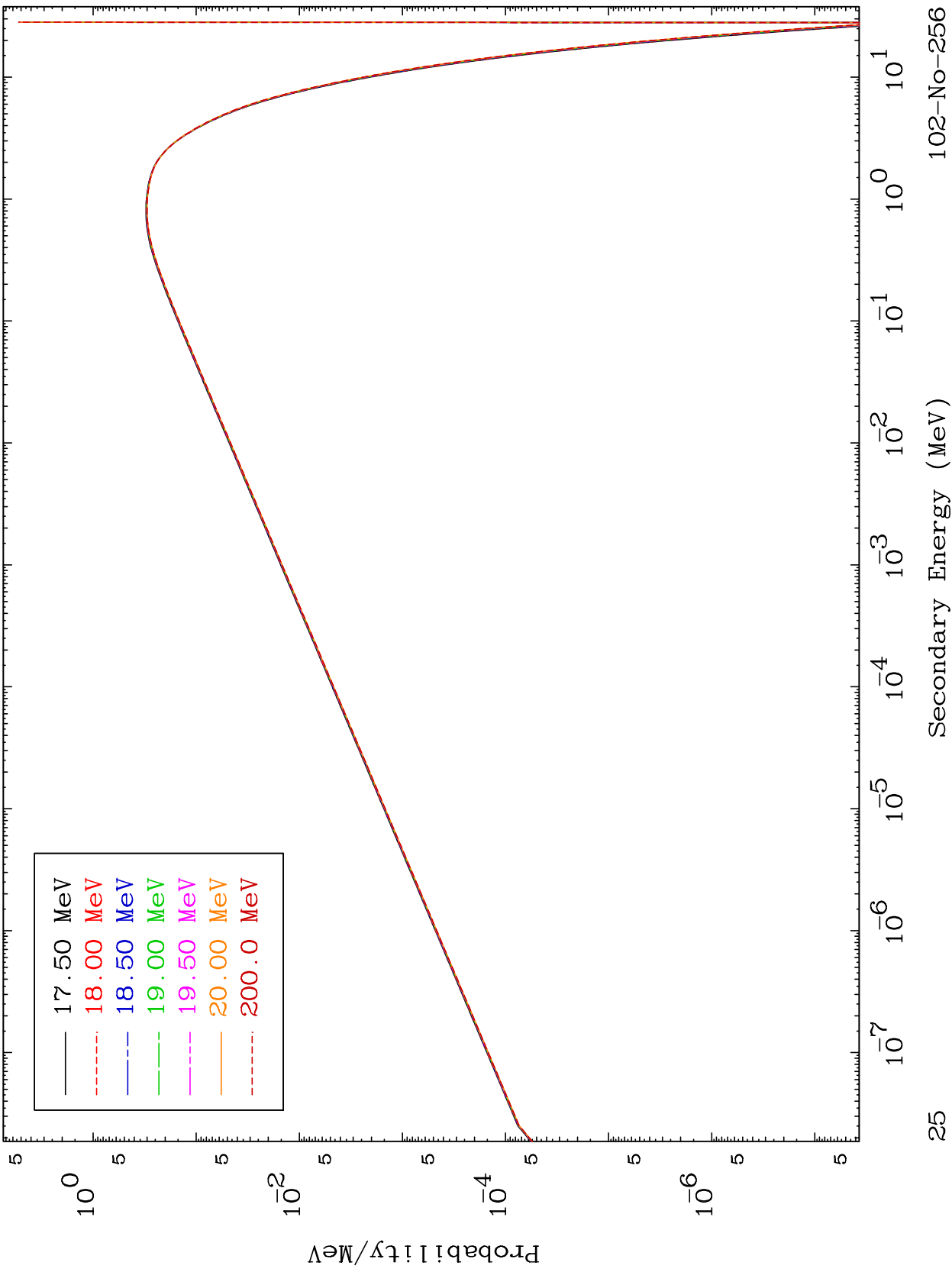








Fission
Energy Distribution

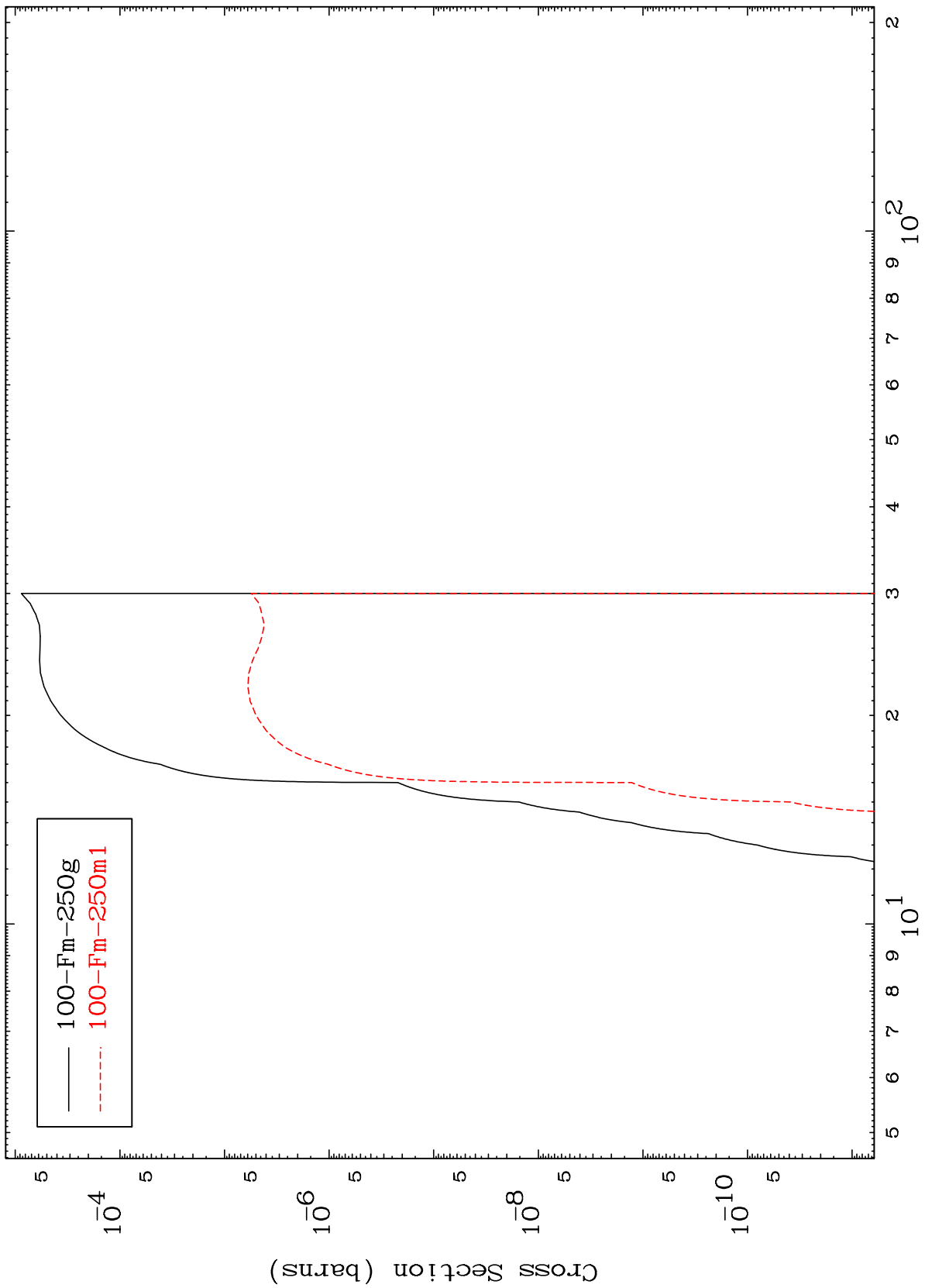


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(n,3n) α

102-No-256

Radionuclide Production Cross Section



26

Incident Energy (MeV)

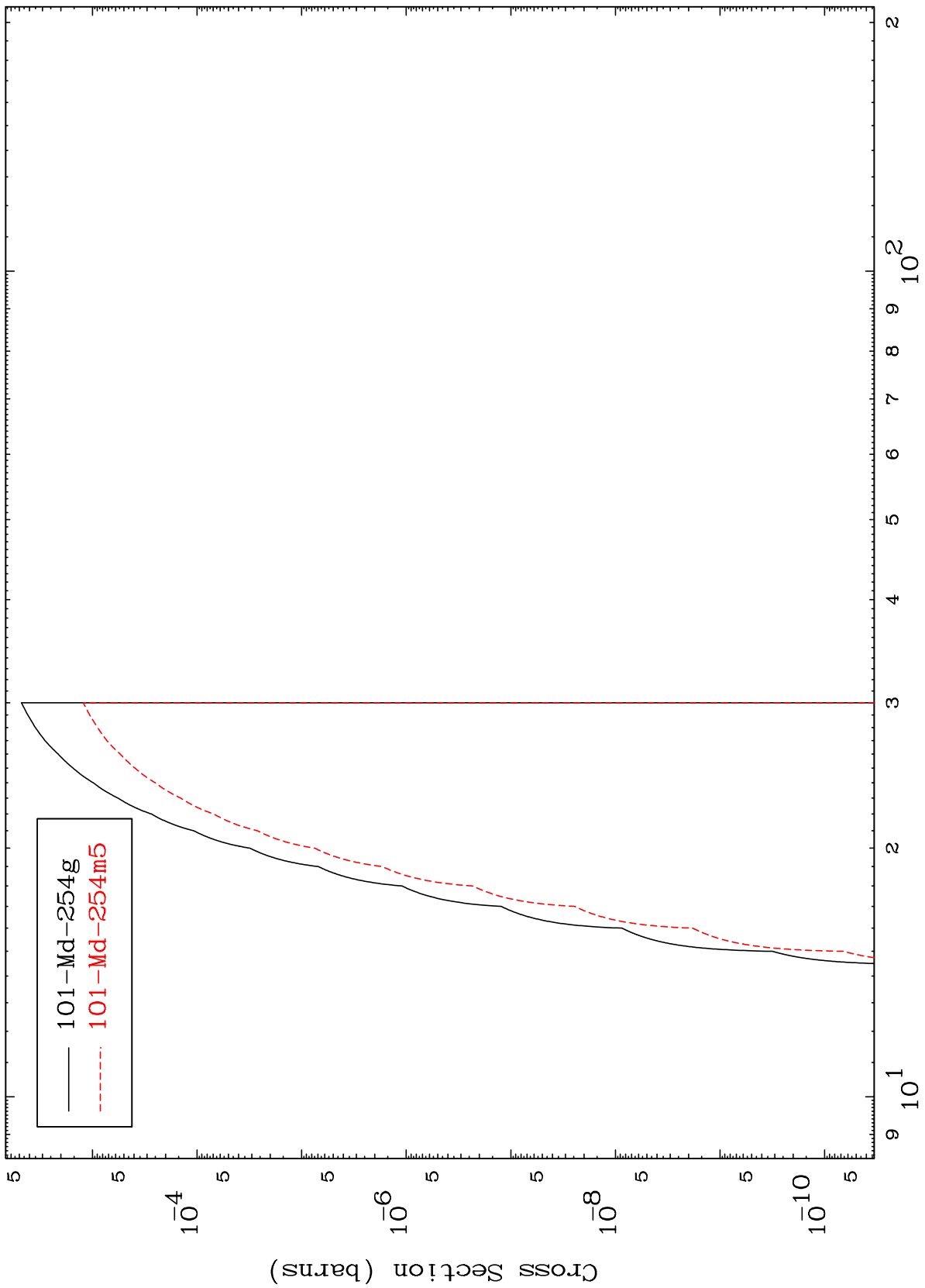
102-No-256

MAT 9974

(n,n') d

102-No-256

Radionuclide Production Cross Section



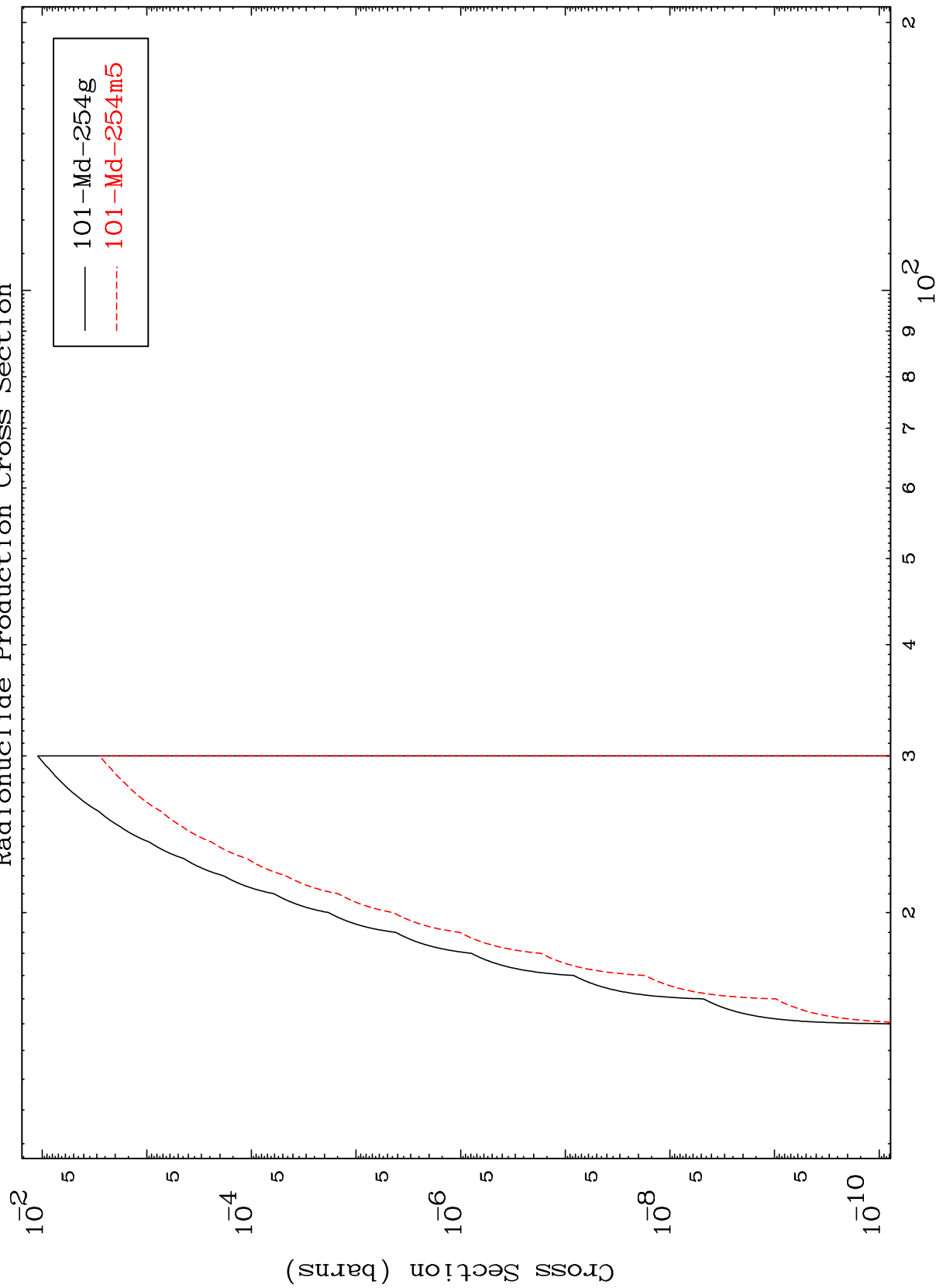
101-Md-254g
101-Md-254m5

27

Incident Energy (MeV)

102-No-256

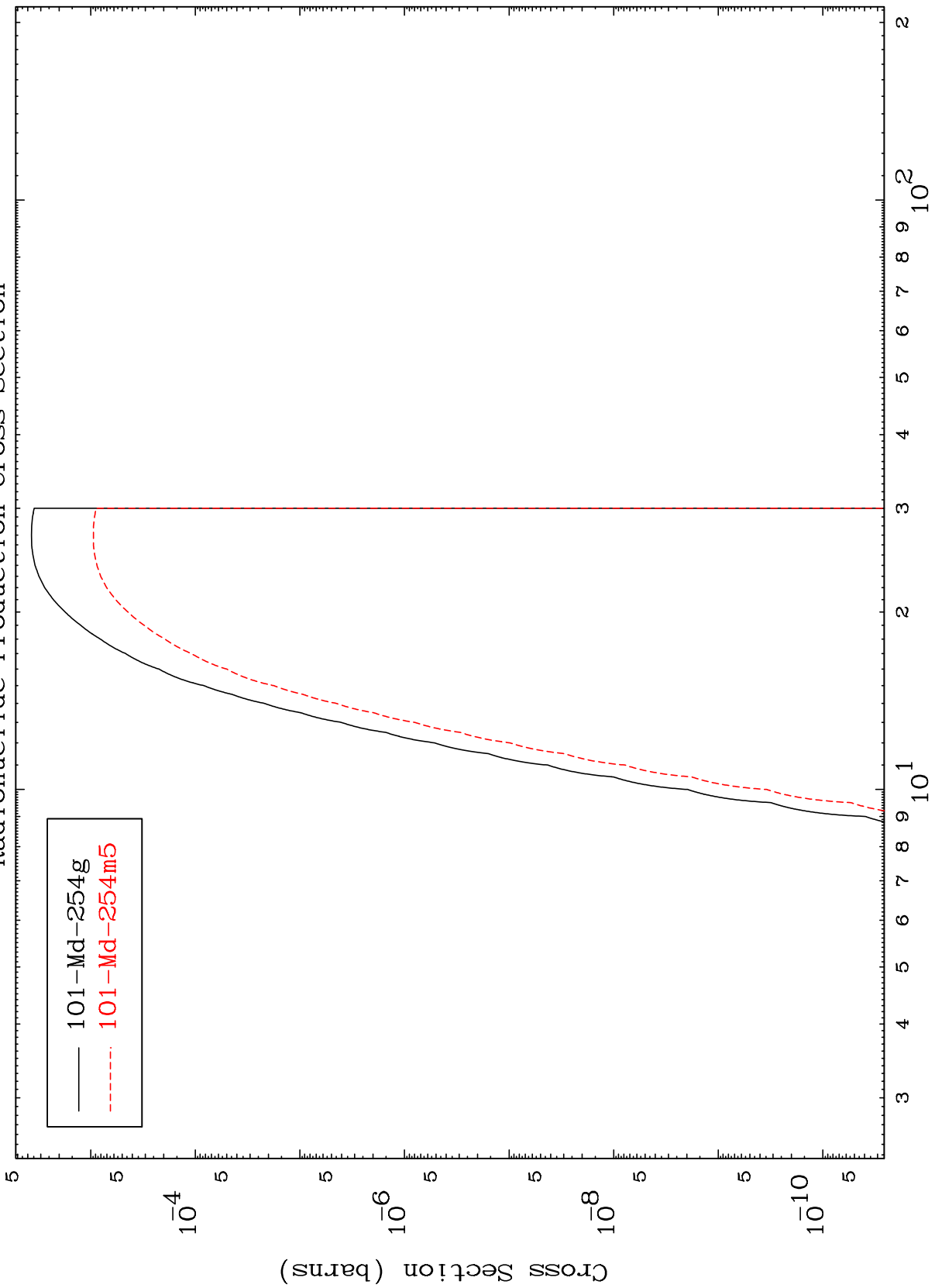
Radionuclide Production Cross Section



MAT 9974

102-No-256

(n, t)
Radionuclide Production Cross Section



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Incident Energy (MeV)

102-No-256