

Program EVALPLOT  
(Version 2015-2)

by

Dermott E. Cullen  
(Present Contact Information)

Dermott E. Cullen  
1466 Hudson Way  
Livermore, CA 94550  
U.S.A.

Tele: 925-443-1911

E.Mail:redcullen1@comcast.net  
Web:home.comcast.net/~redcullen1

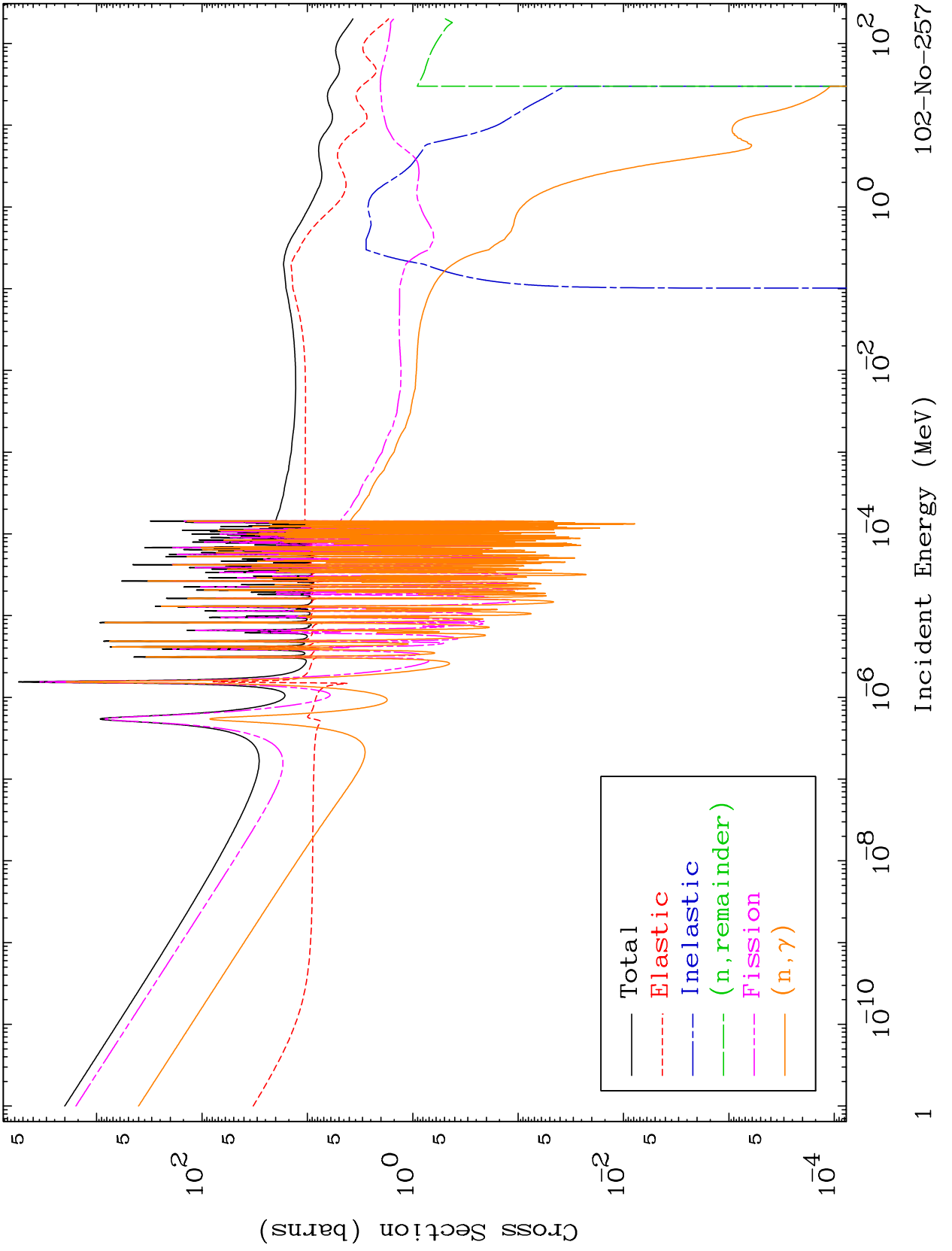
Press Mouse Button to Start

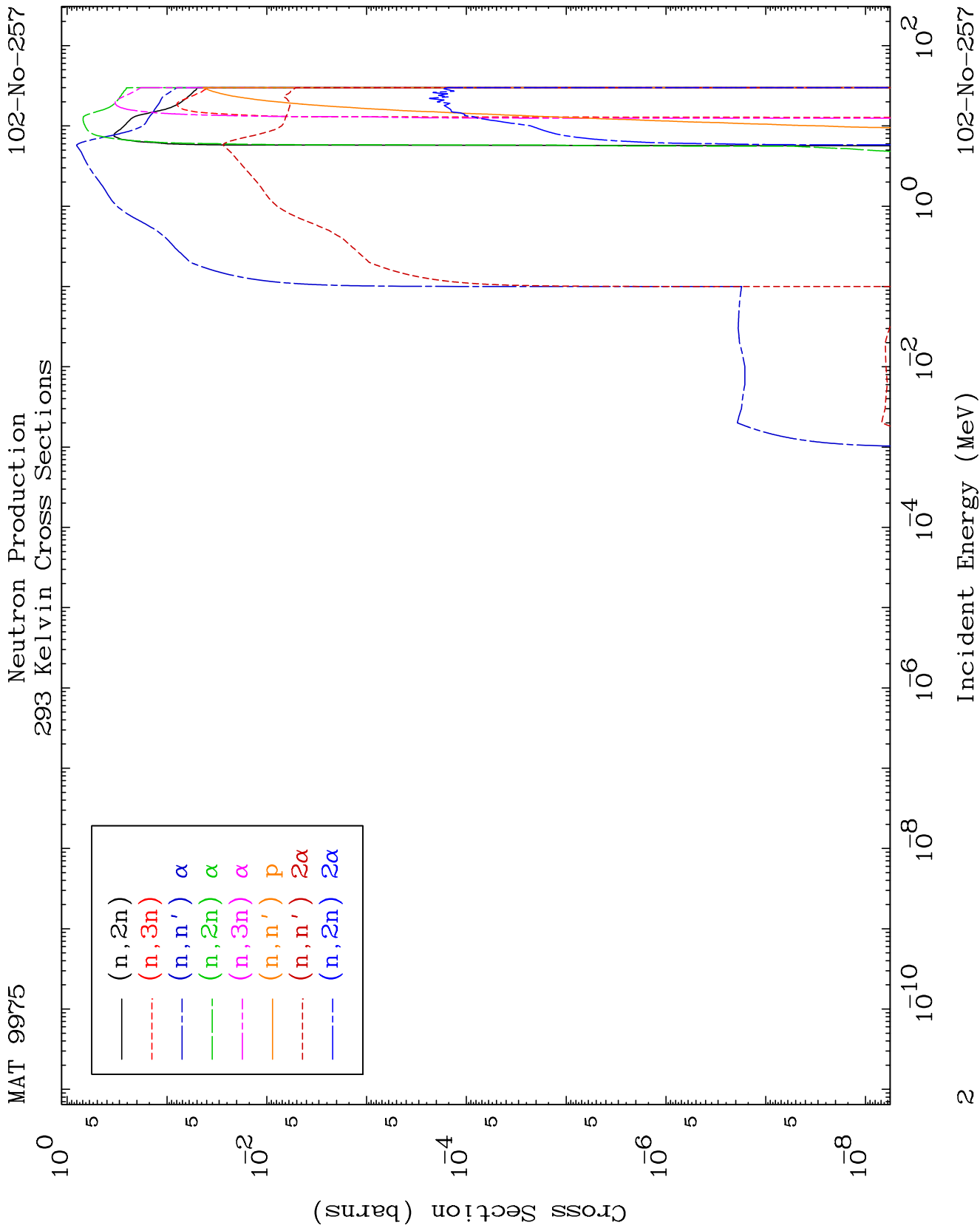
MAT 9975

Major

293 Kelvin Cross Sections

102-No-257

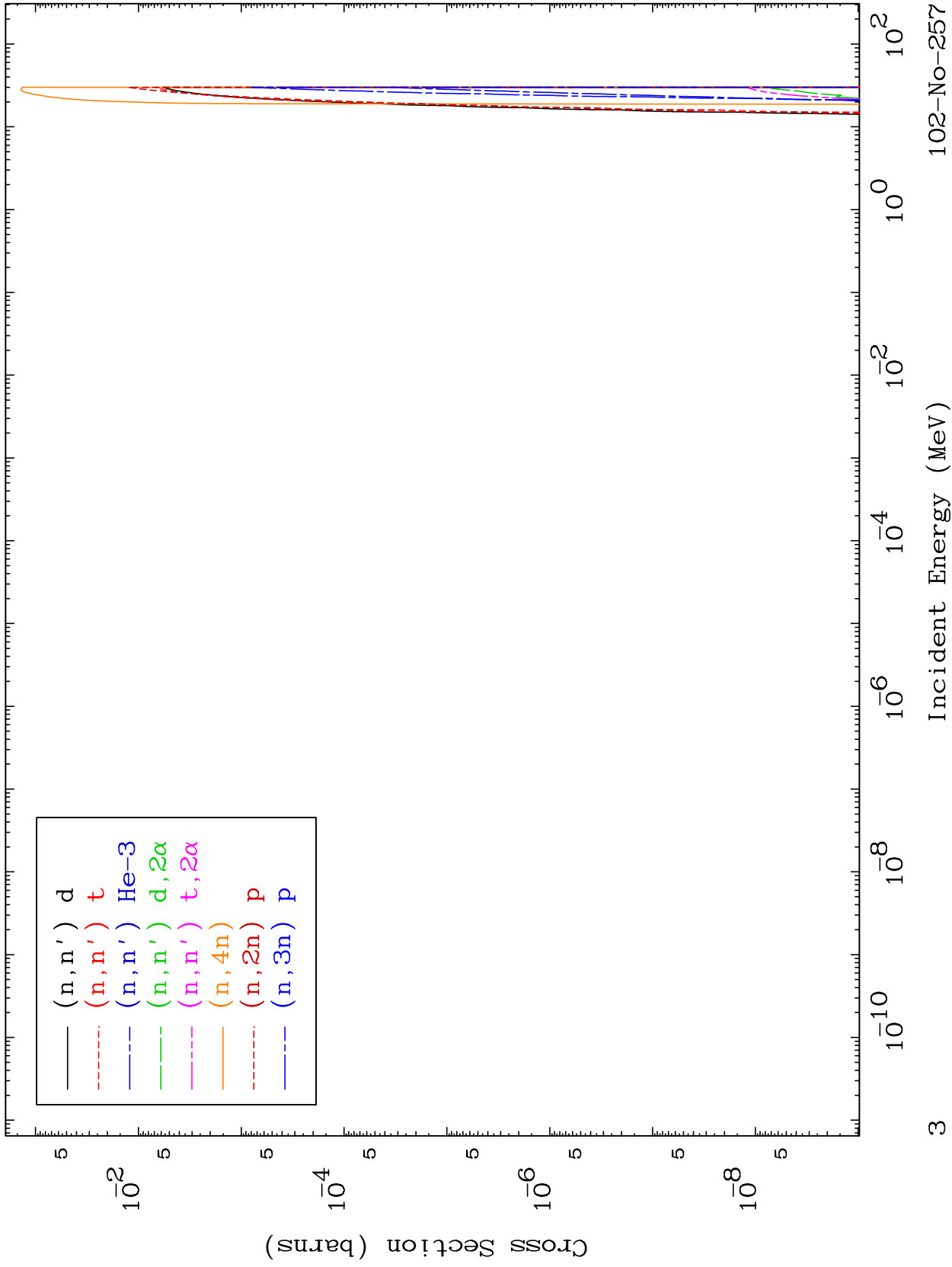


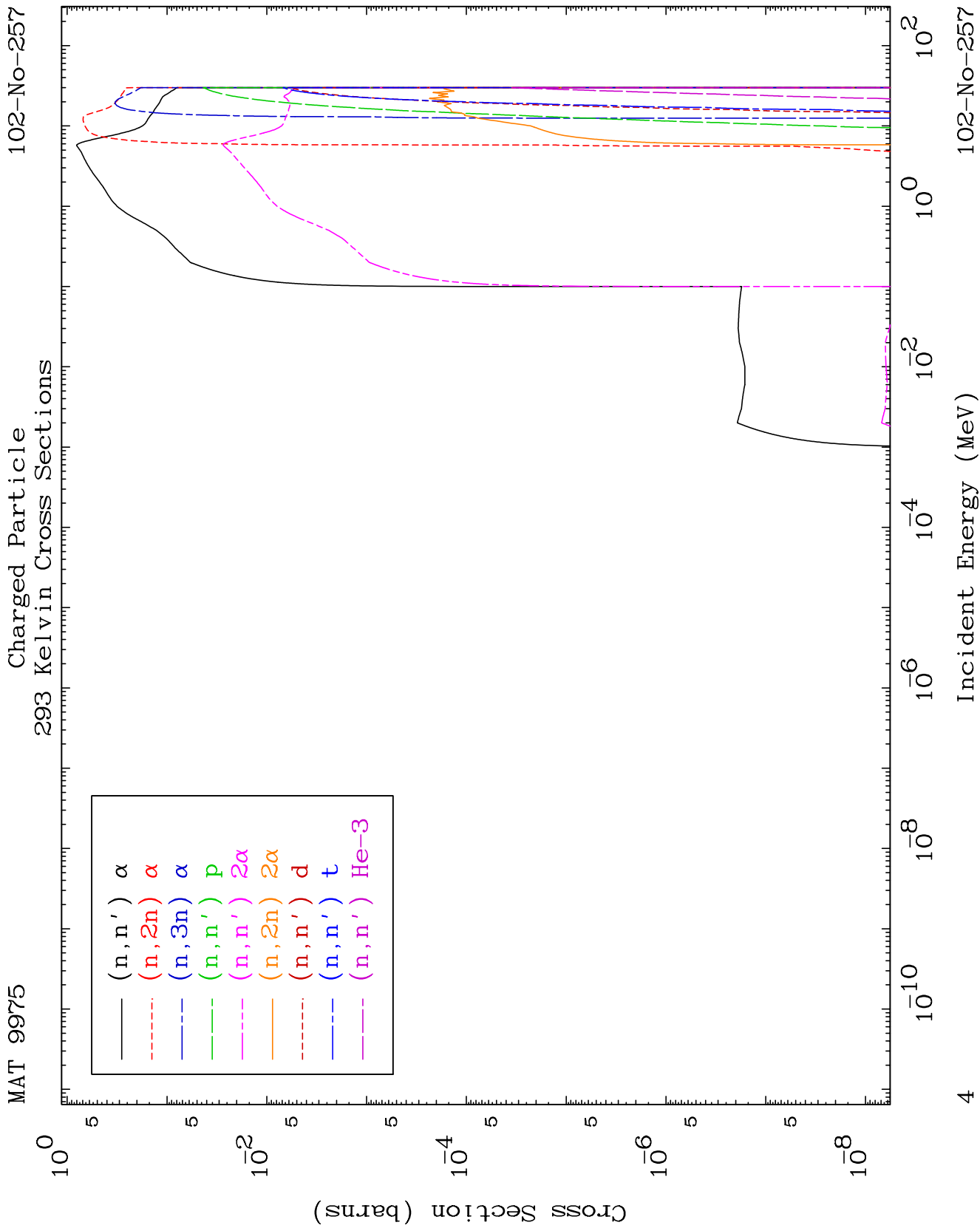


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Neutron Production  
293 Kelvin Cross Sections

102-No-257

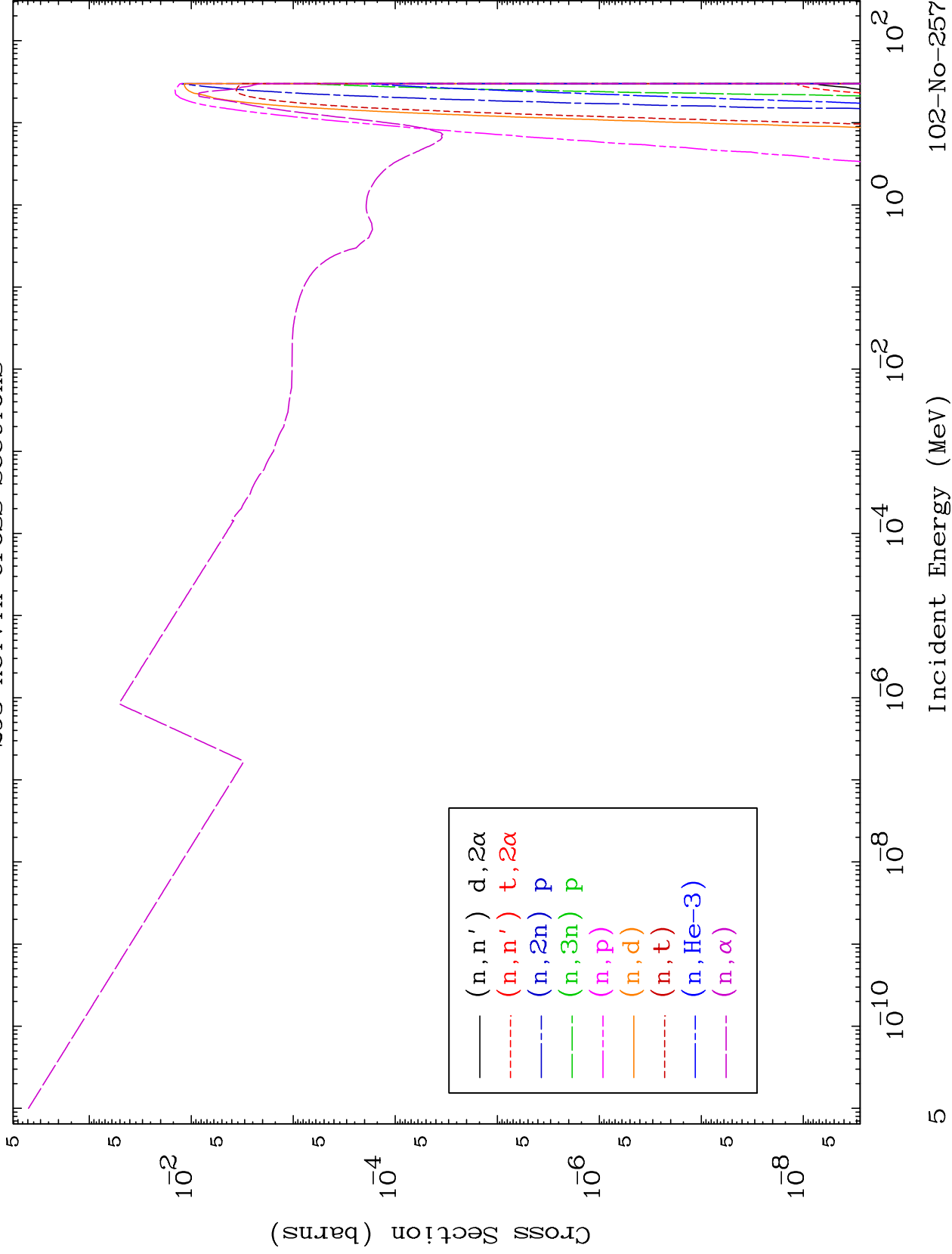




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Charged Particle  
293 Kelvin Cross Sections

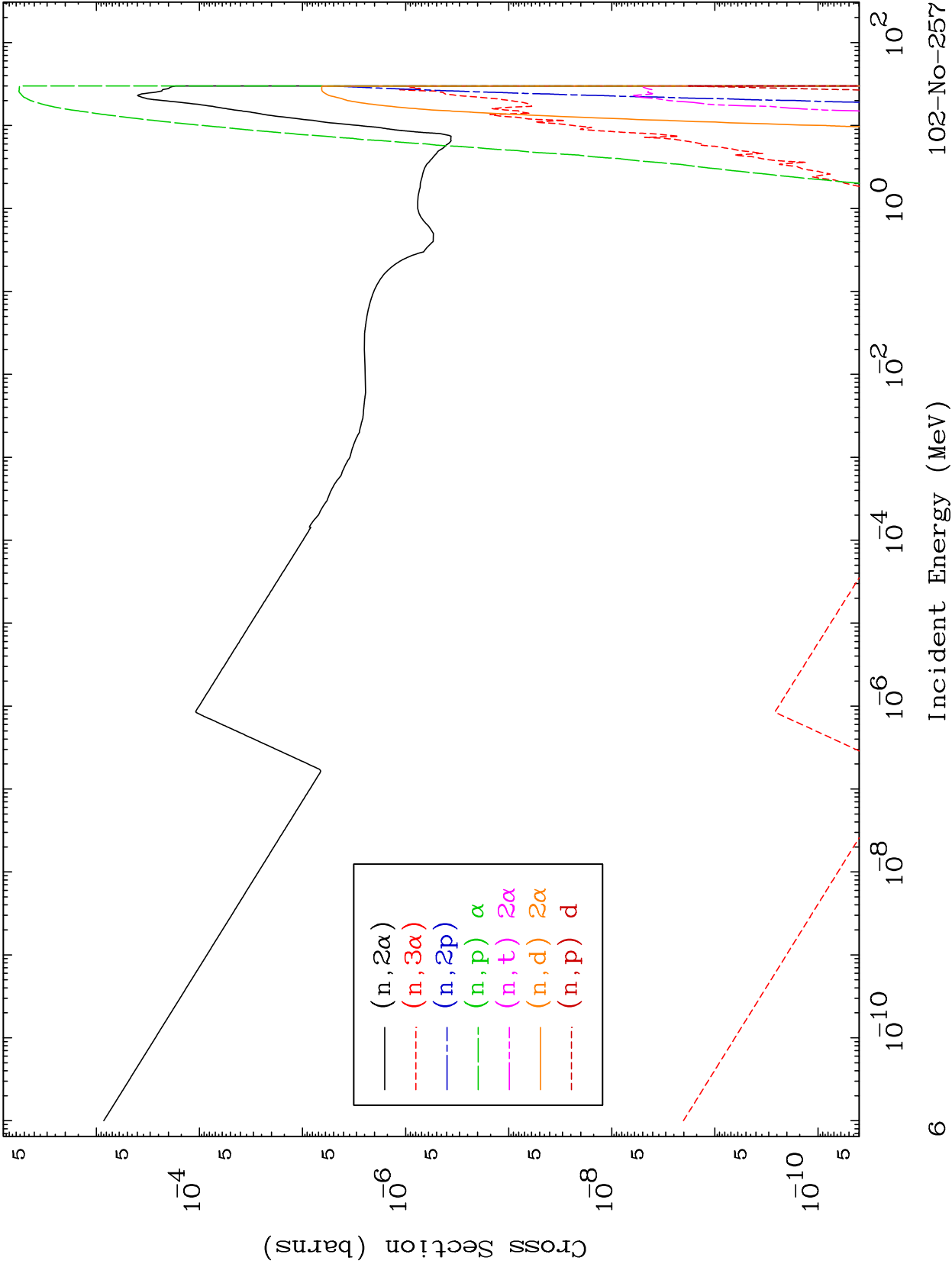
102-No-257

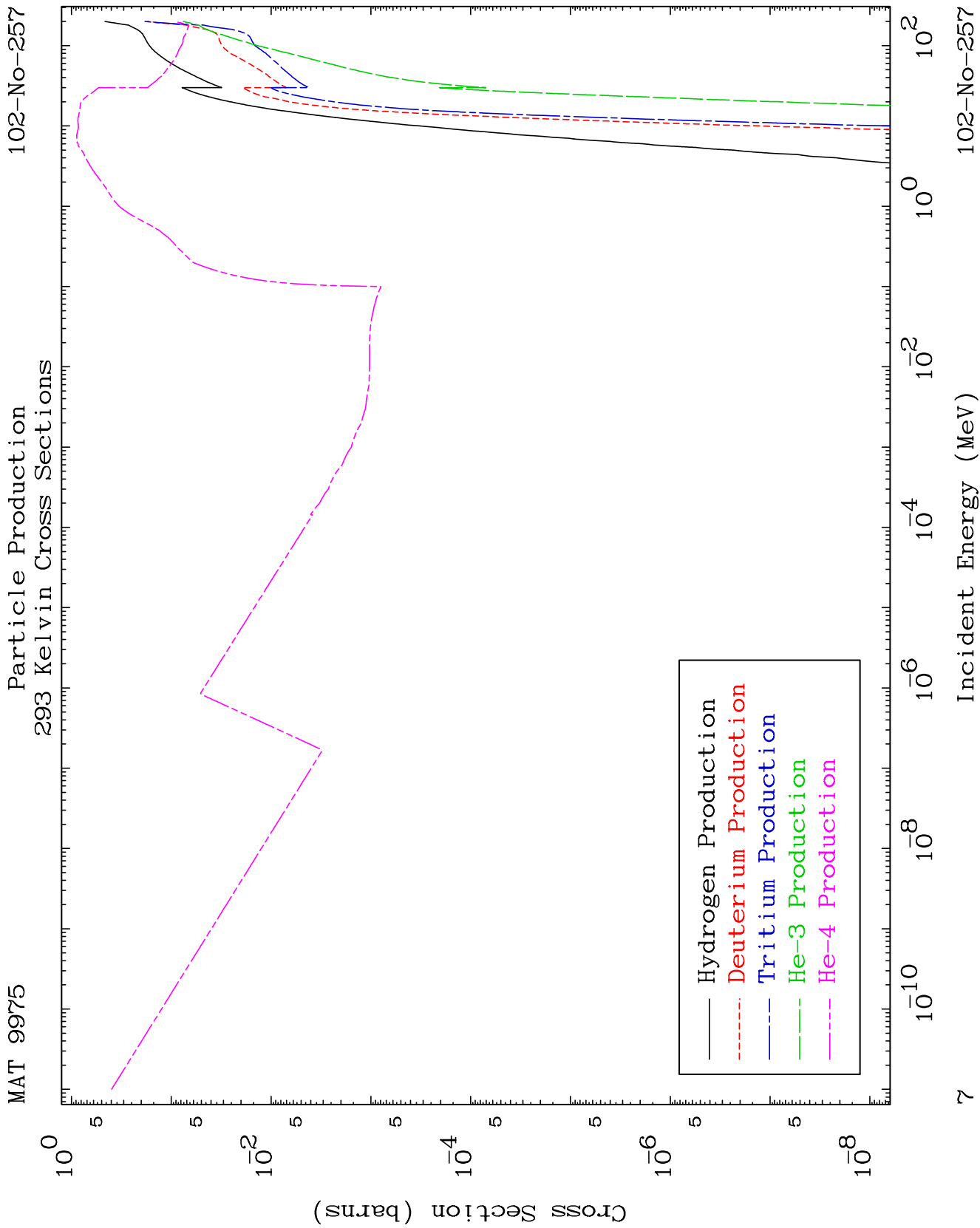


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Charged Particle  
293 Kelvin Cross Sections

102-No-257



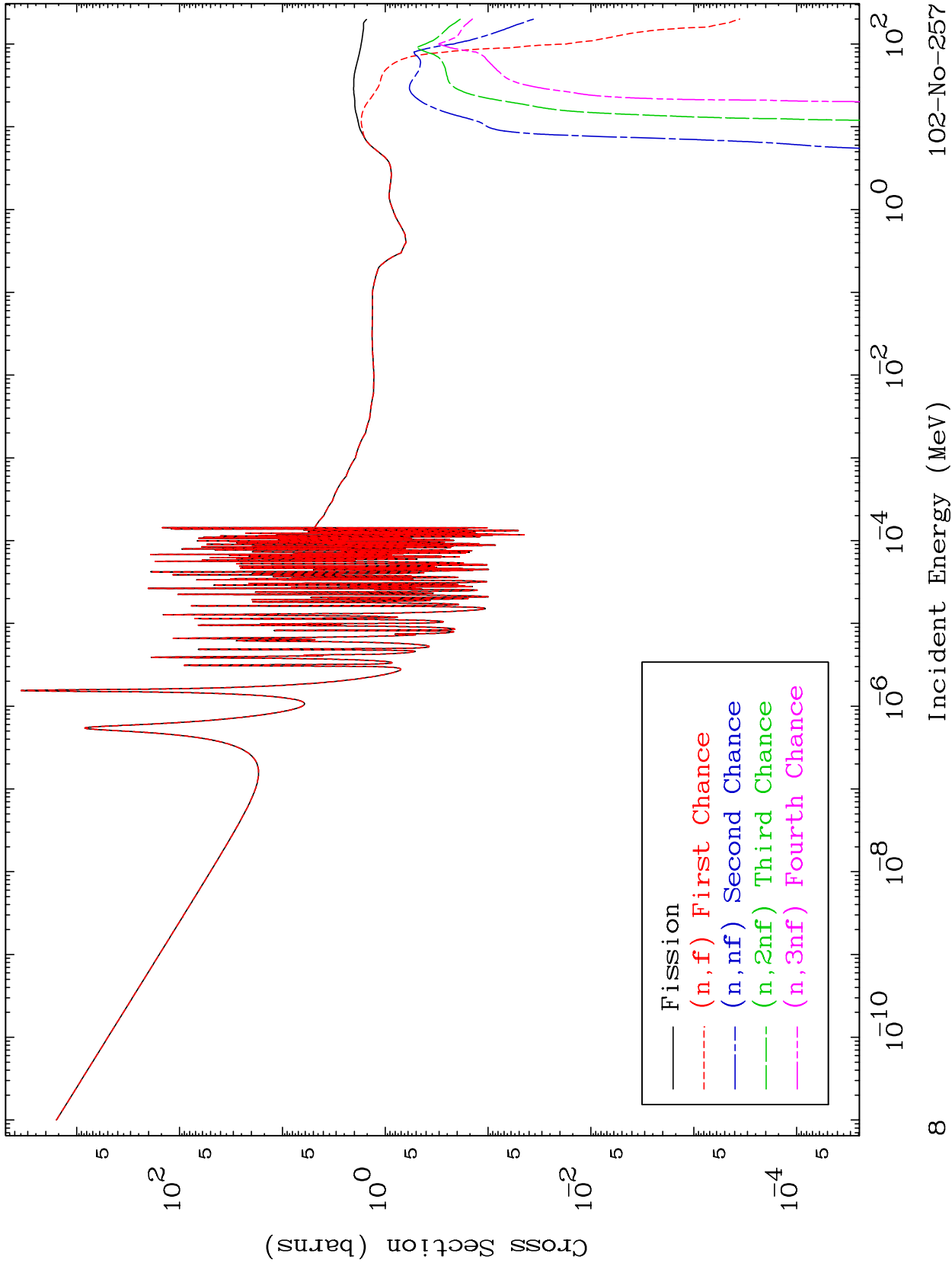


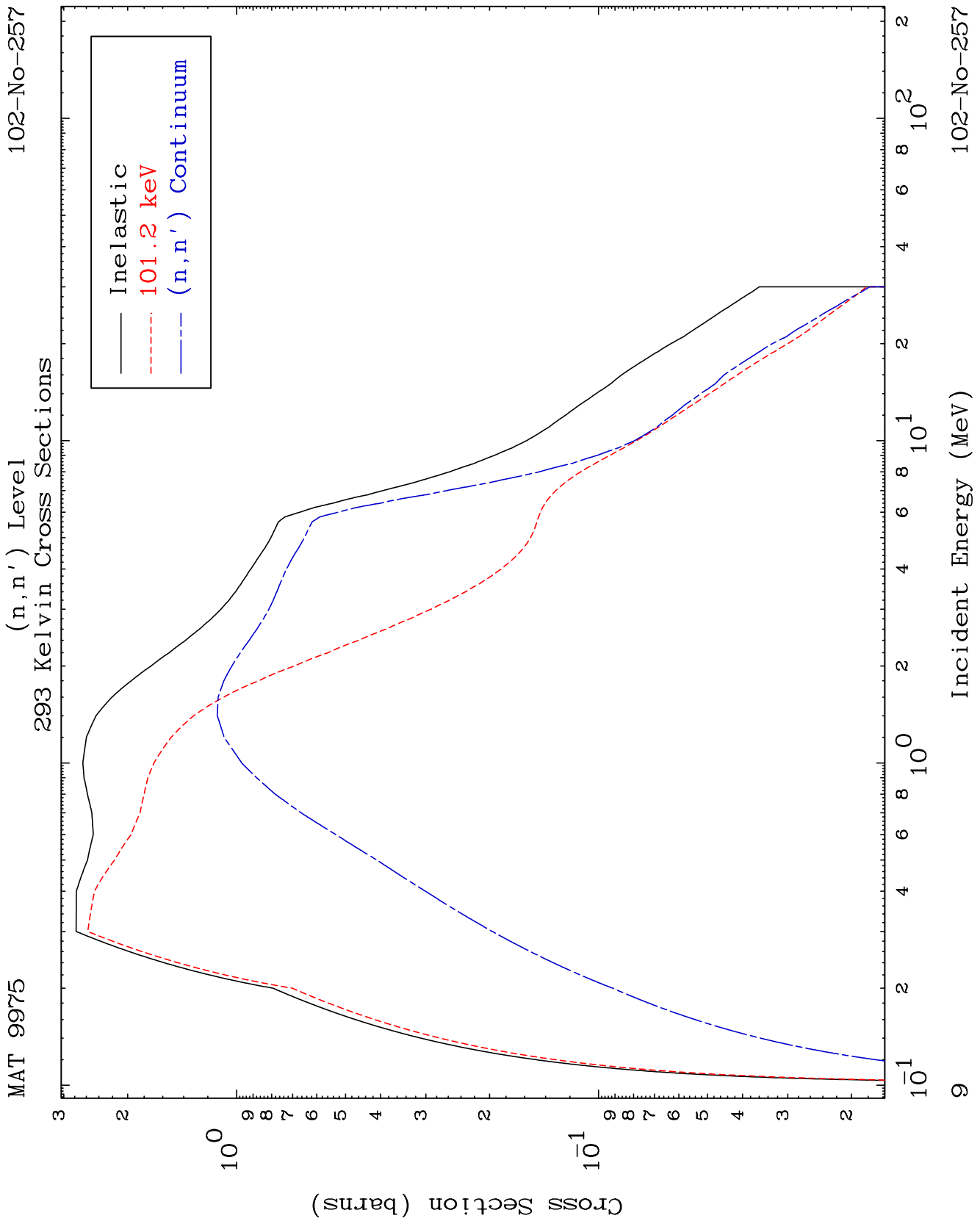


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Fission  
293 Kelvin Cross Sections

102-No-257

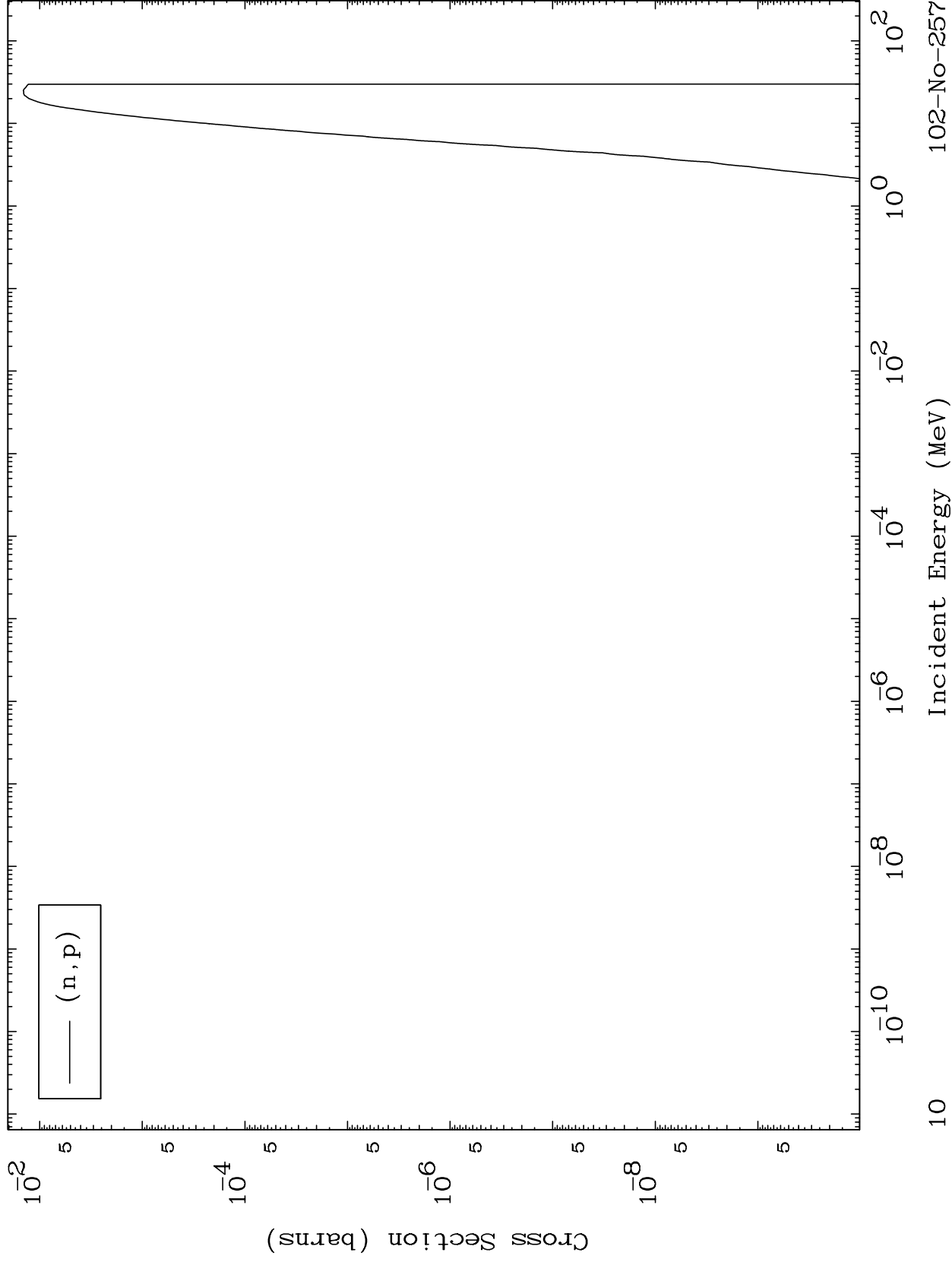




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(n,p) Levels  
293 Kelvin Cross Sections

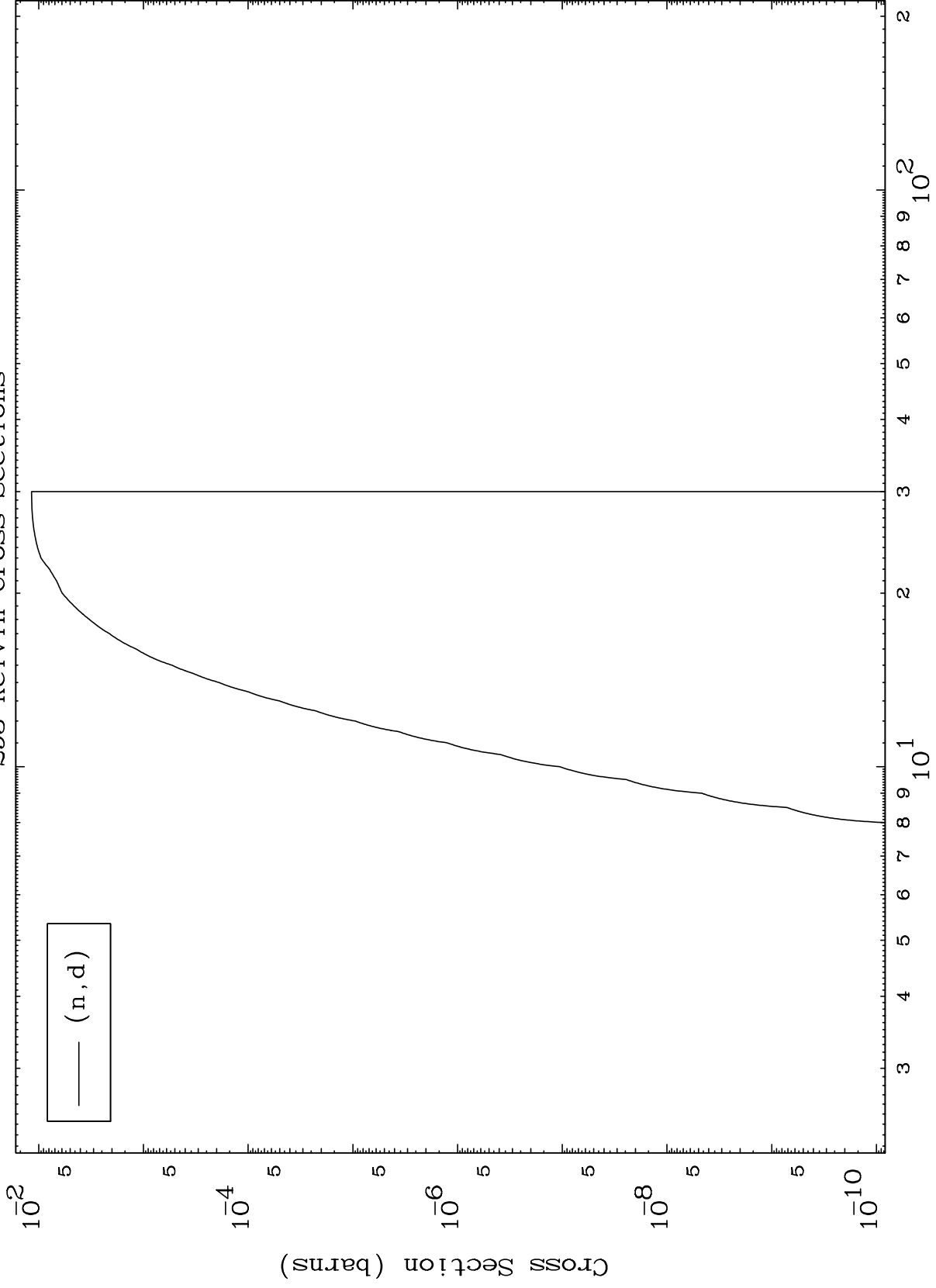
102-No-257



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(n,d) Levels  
293 Kelvin Cross Sections

102-No-257

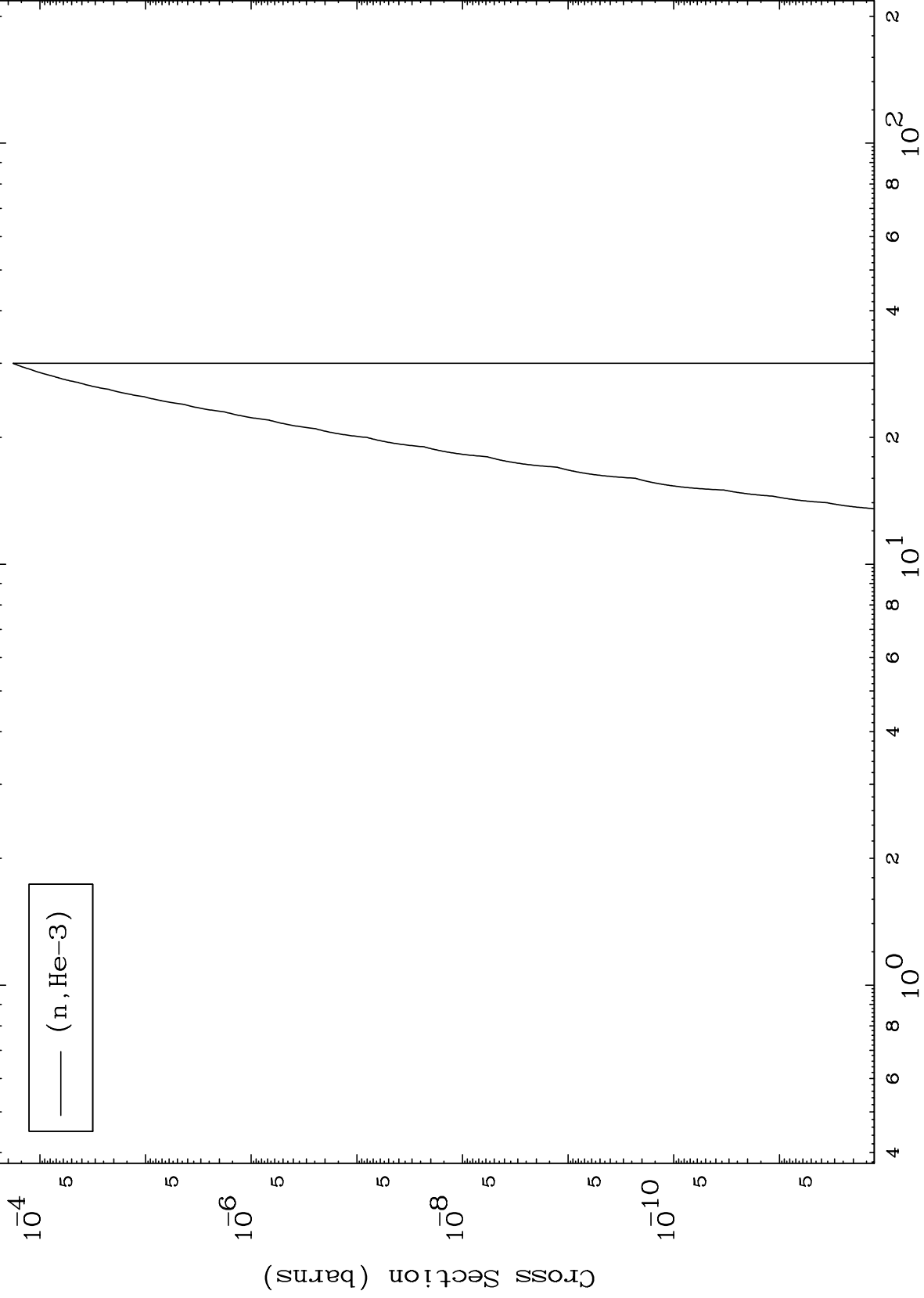




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(n,He3) Levels  
293 Kelvin Cross Sections

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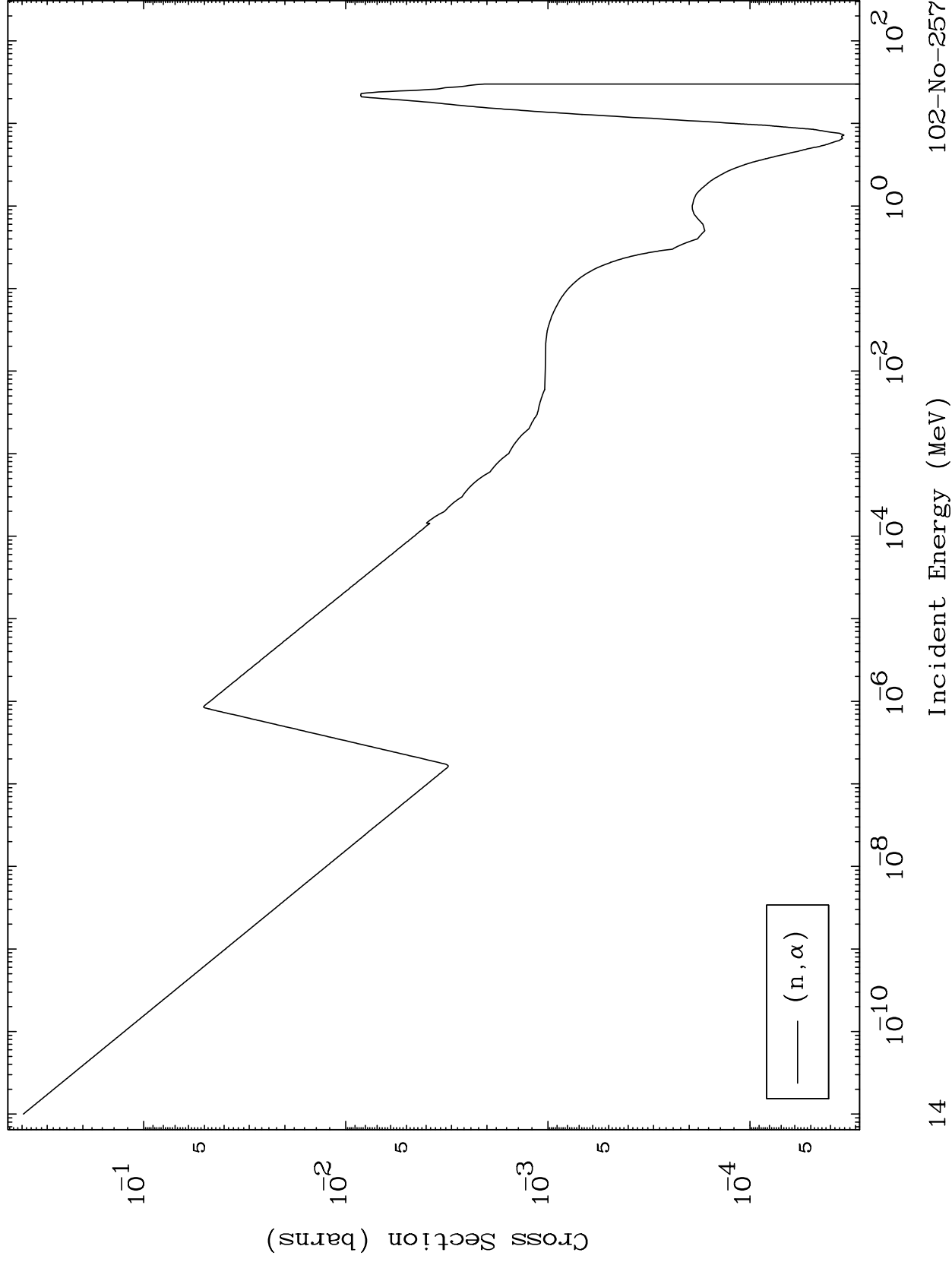
Incident Energy (MeV)

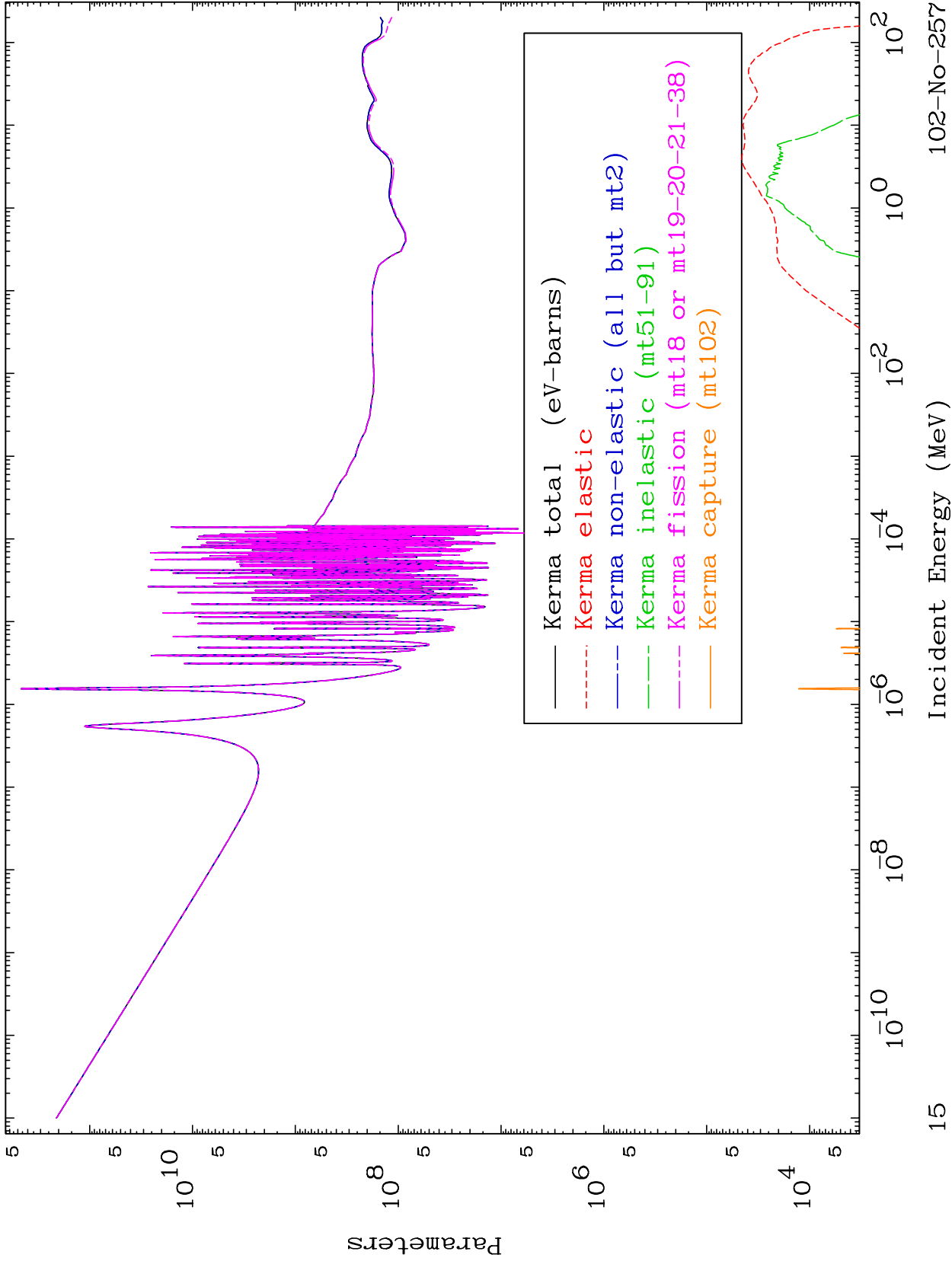
102-No-257

MAT 9975

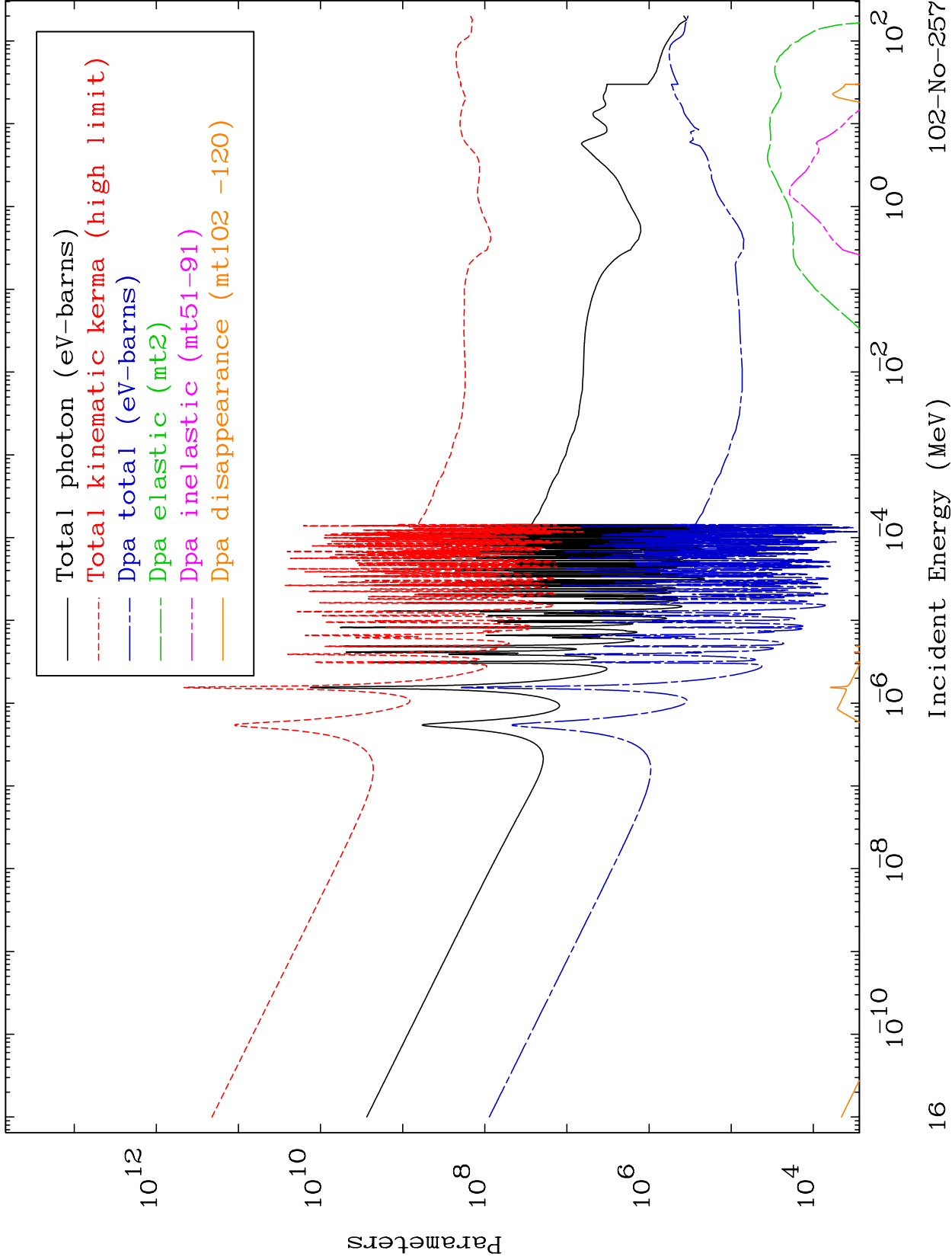
(n,α) Levels  
293 Kelvin Cross Sections

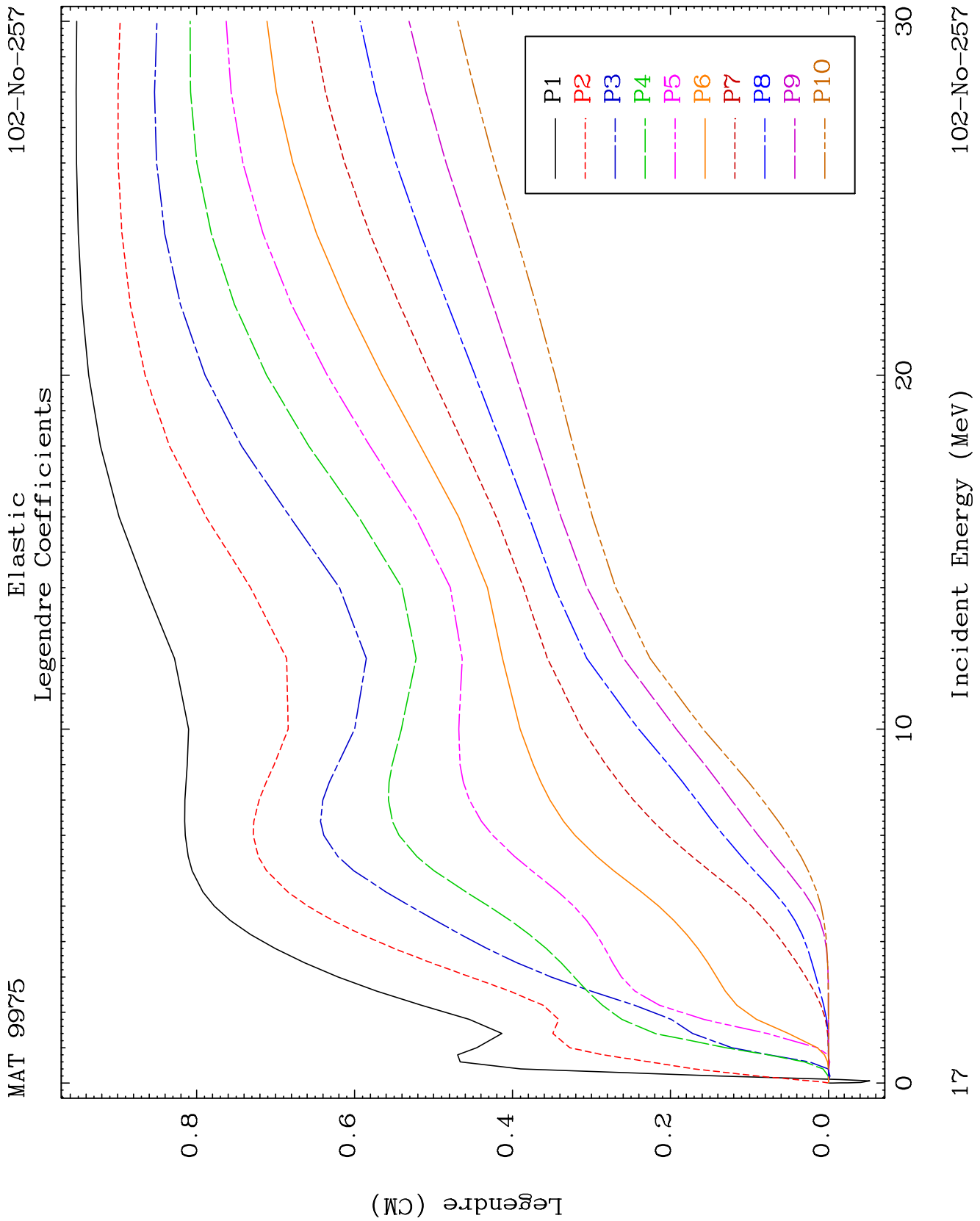
102-No-257

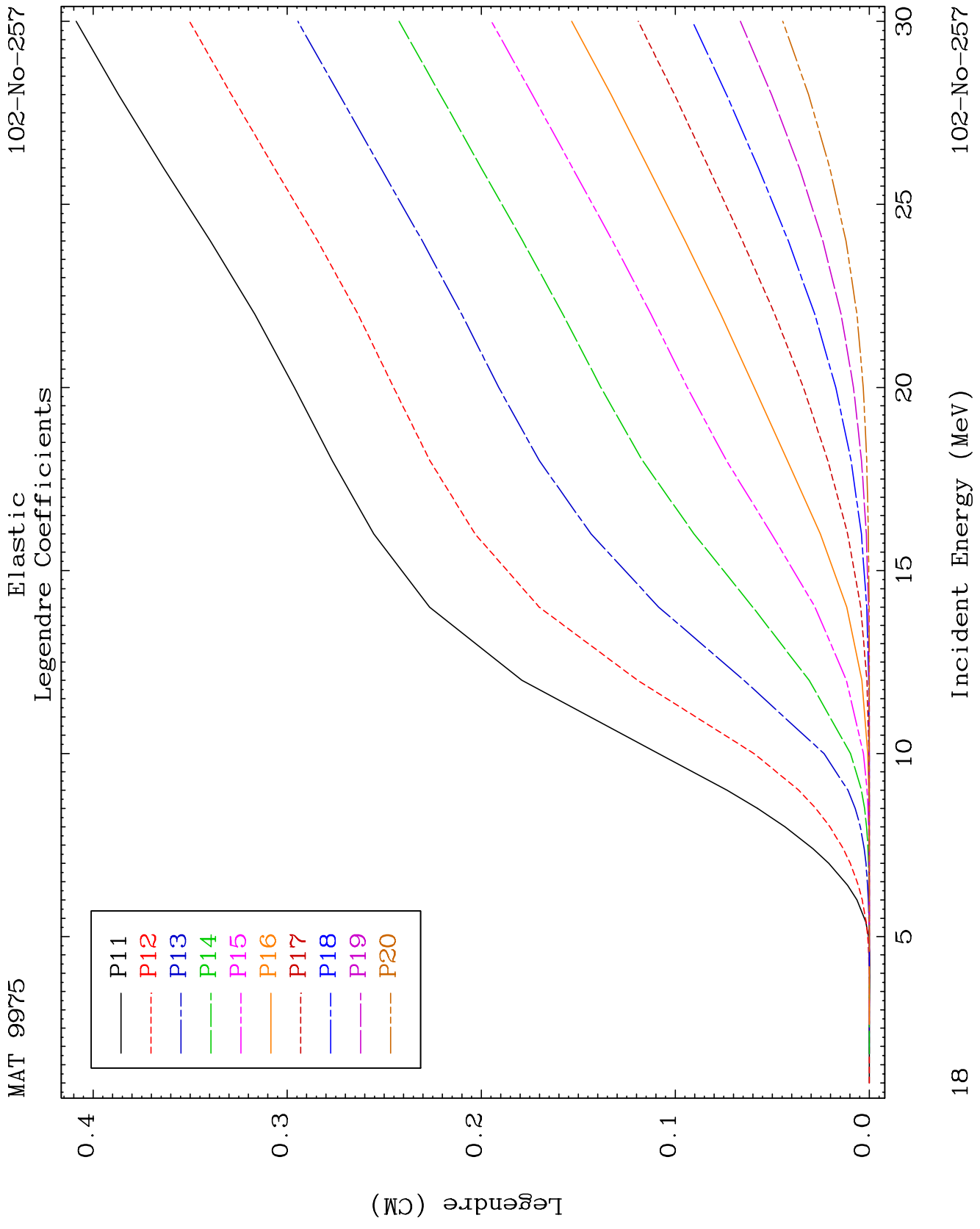












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Elastic Legendre Coefficients

102-No-257

18

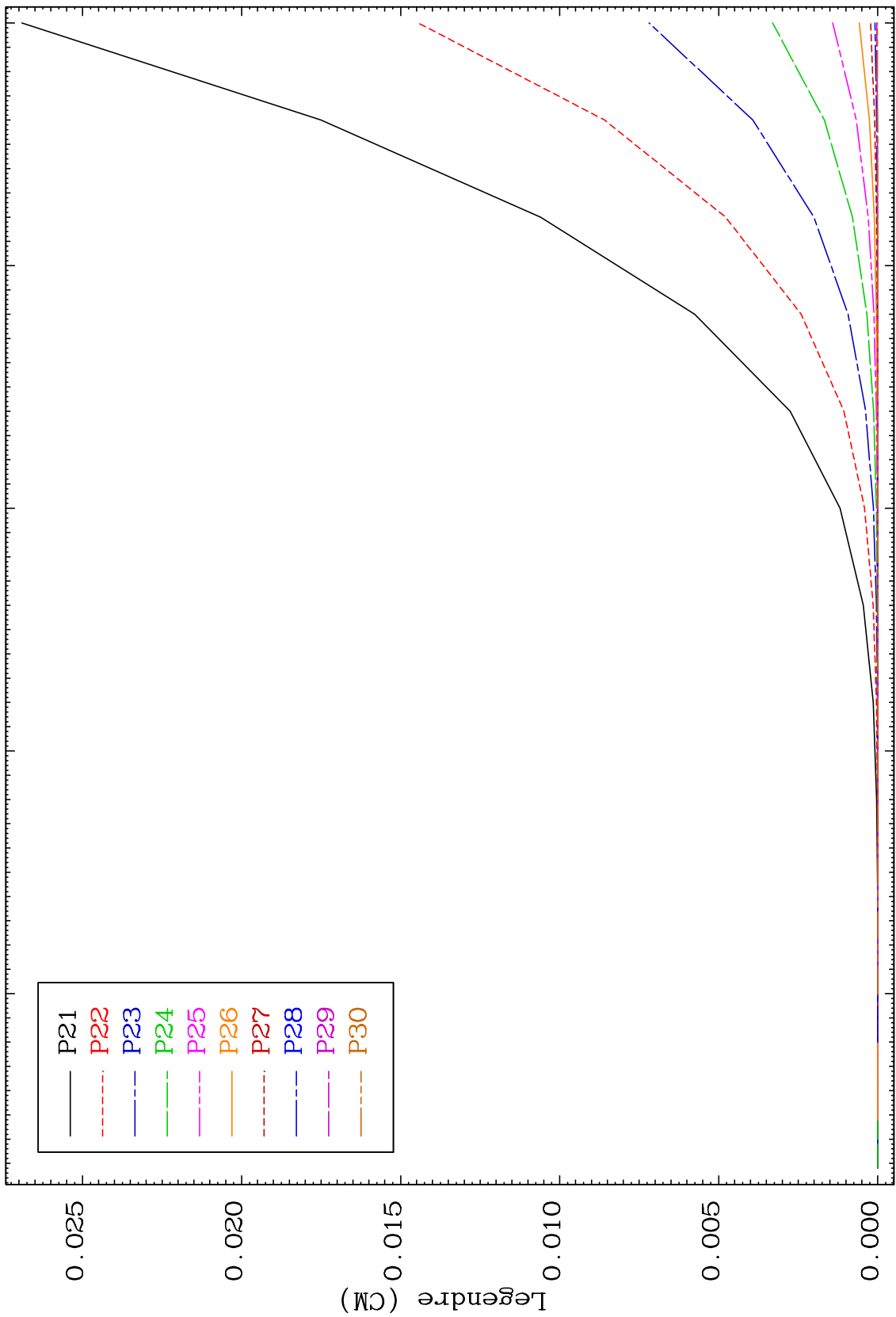
Incident Energy (MeV)

102-No-257

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Elastic  
Legendre Coefficients

102-No-257



19

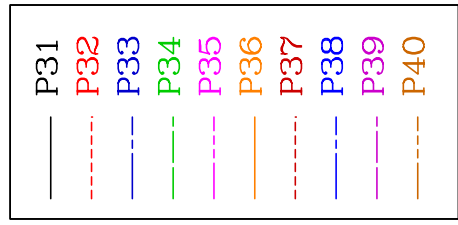
Incident Energy (MeV)

102-No-257

MAT 9975

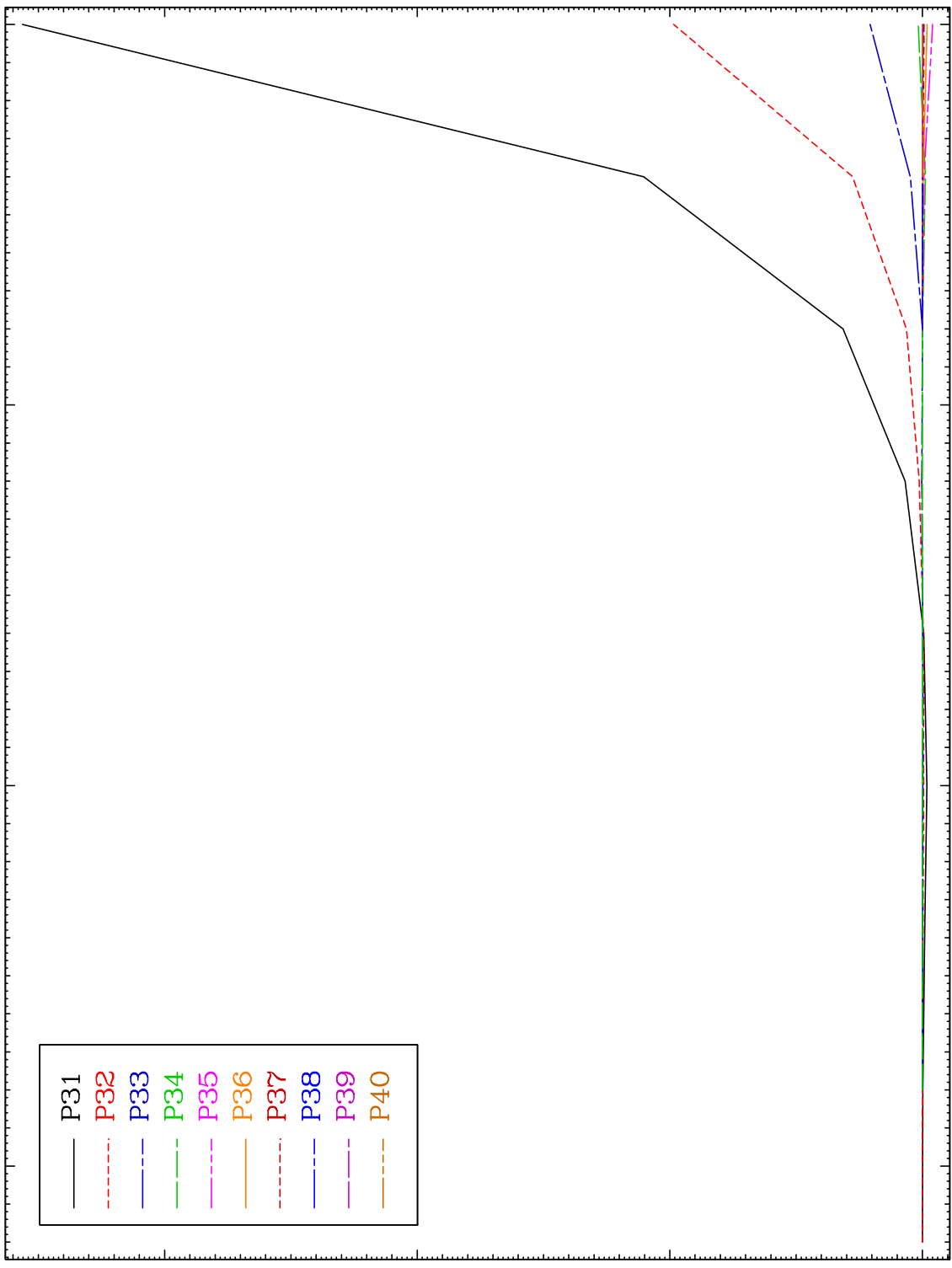
Elastic Legendre Coefficients

102-No-257



$\times 10^{-6}$

Legendre (CM)



30

25

20

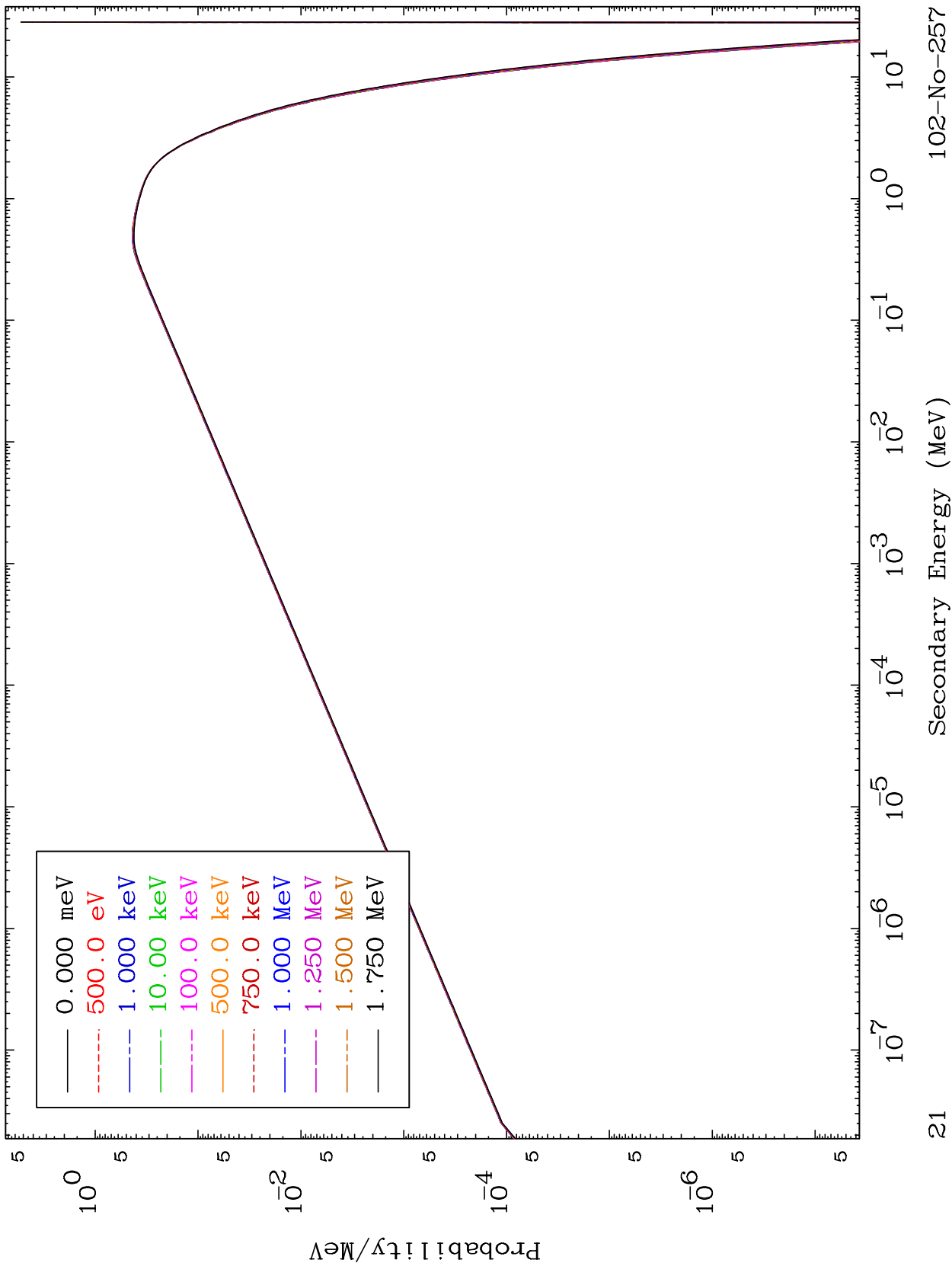
15

20

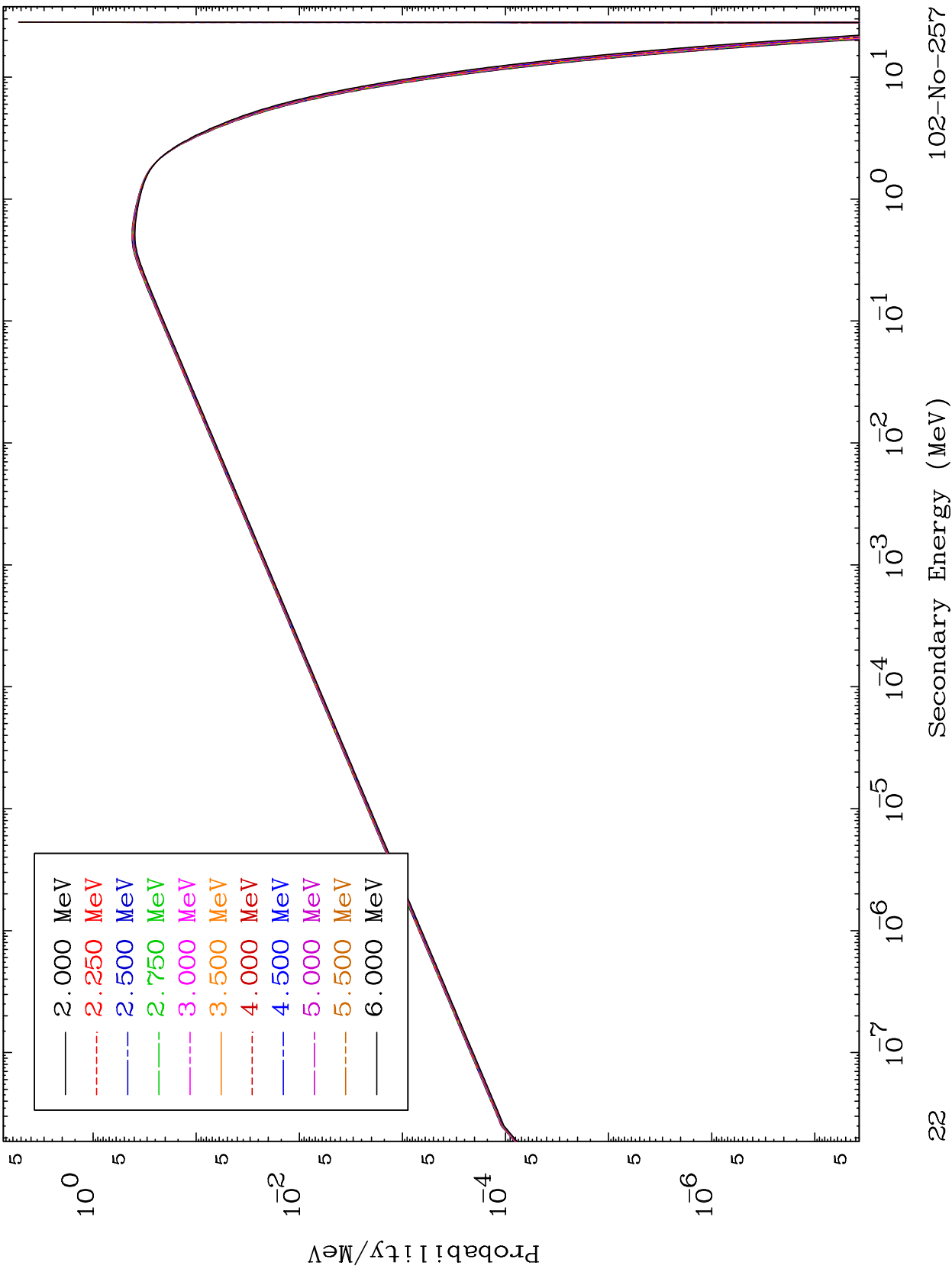
Incident Energy (MeV)

102-No-257

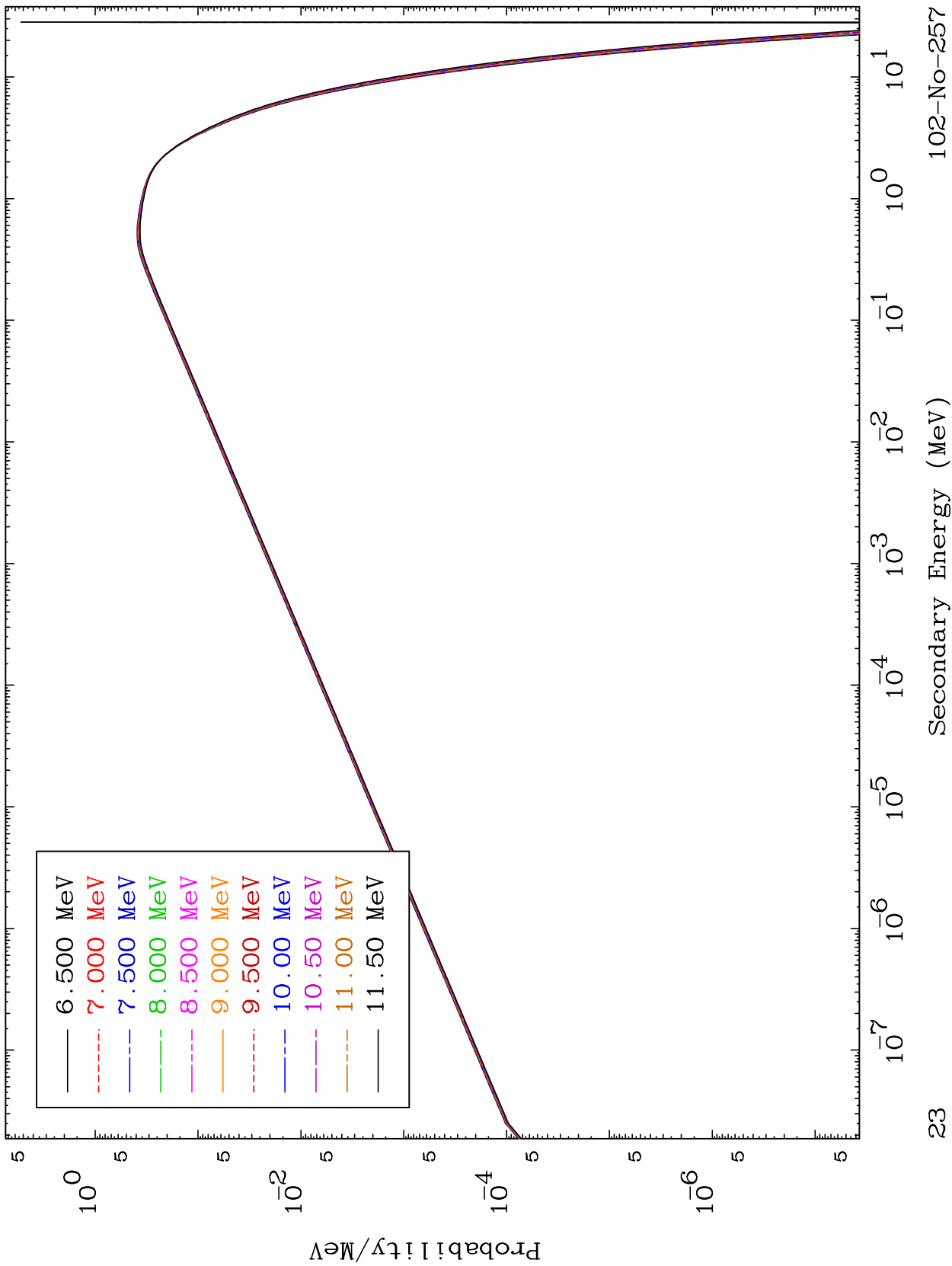
Fission  
Energy Distribution



Fission Energy Distribution

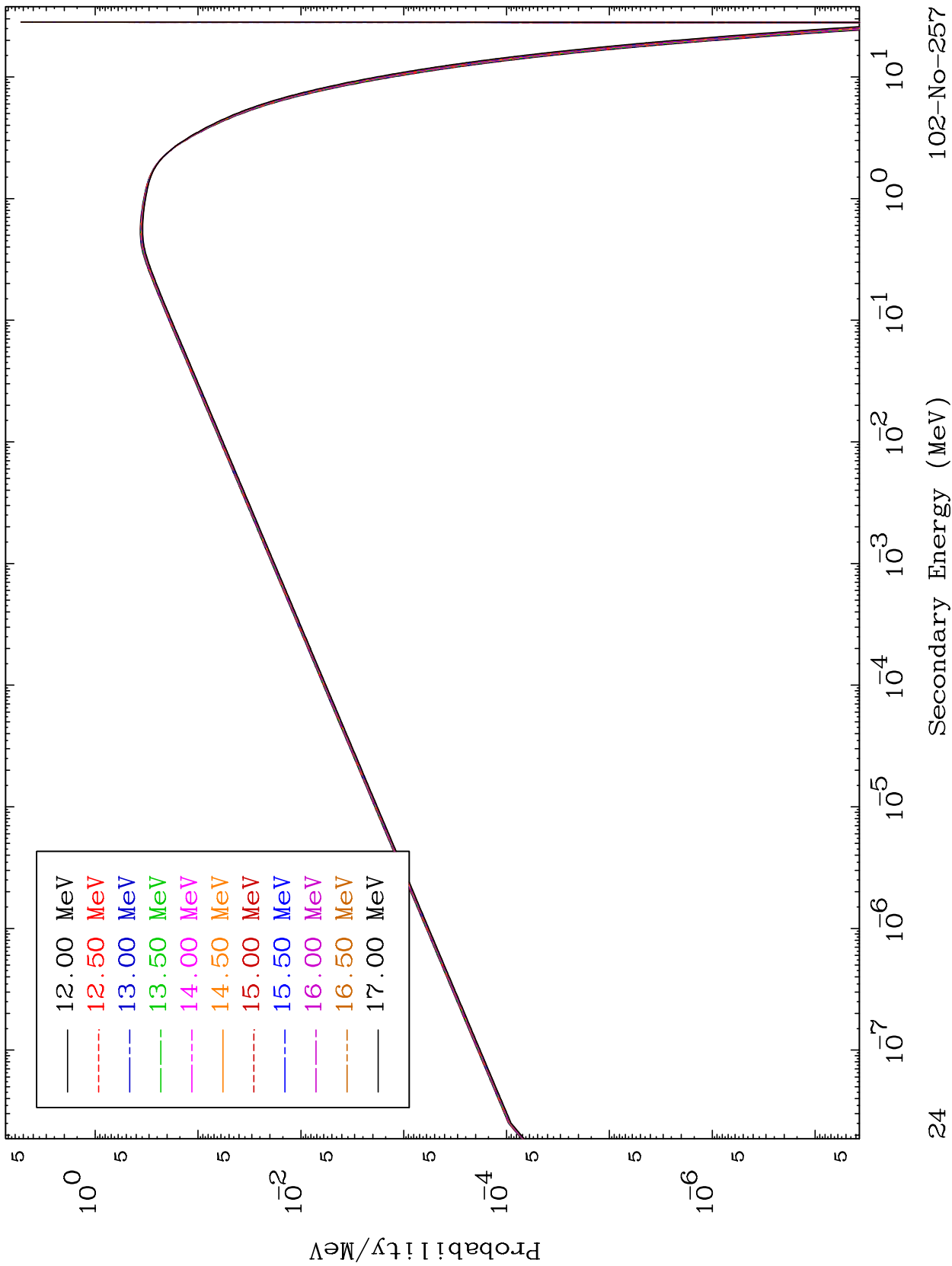


Fission Energy Distribution

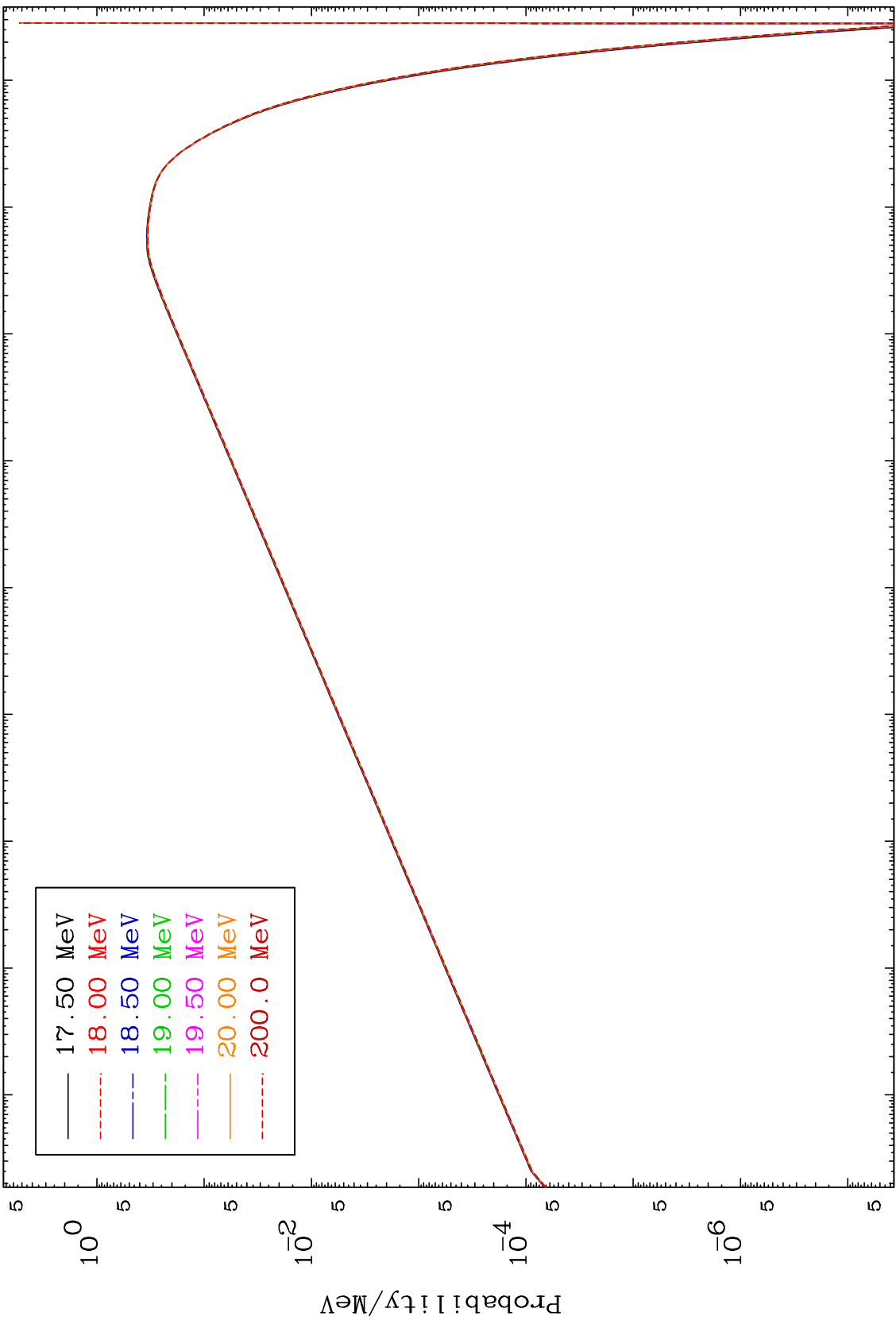




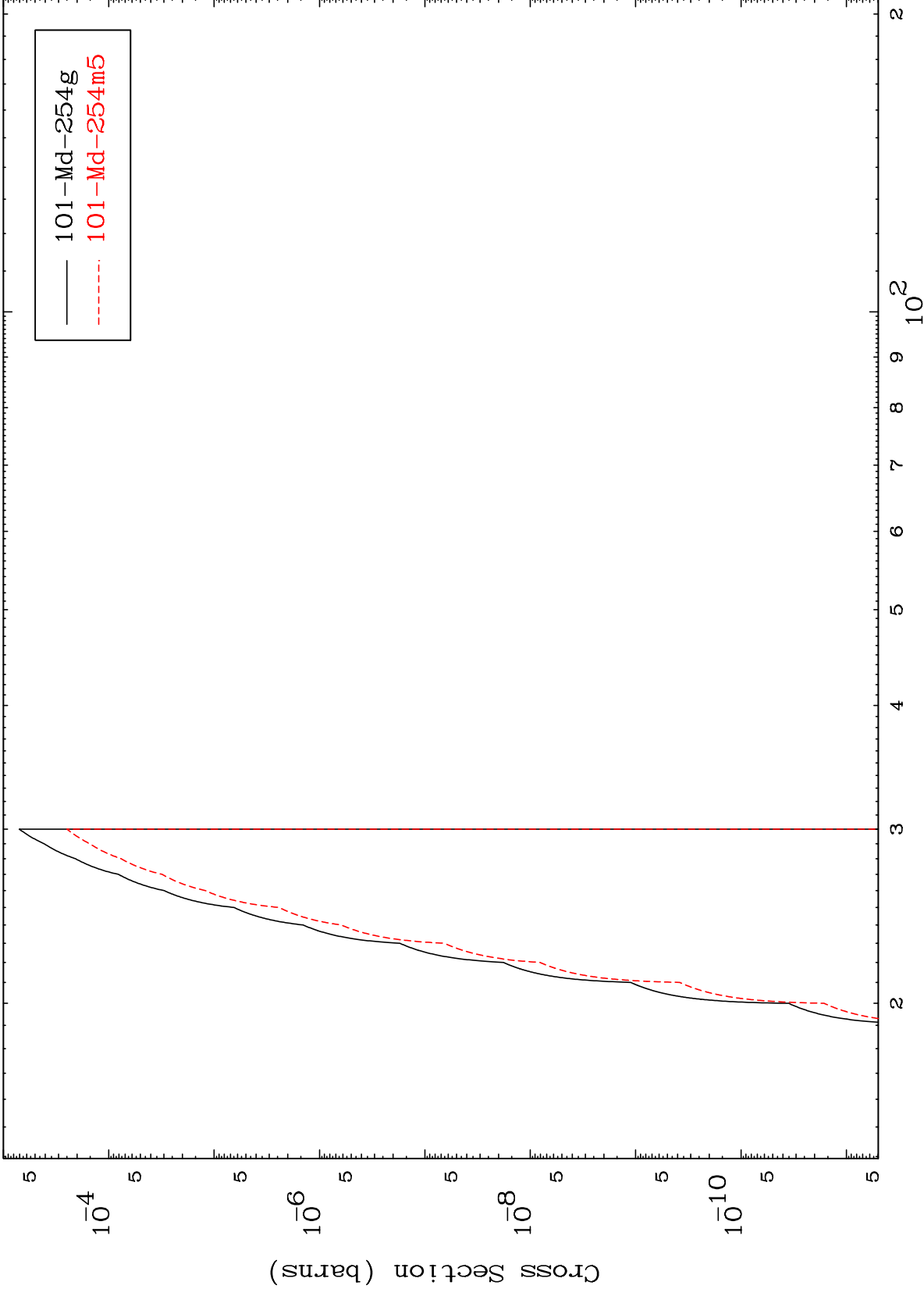
Fission  
Energy Distribution



Fission  
Energy Distribution



Radionuclide Production Cross Section

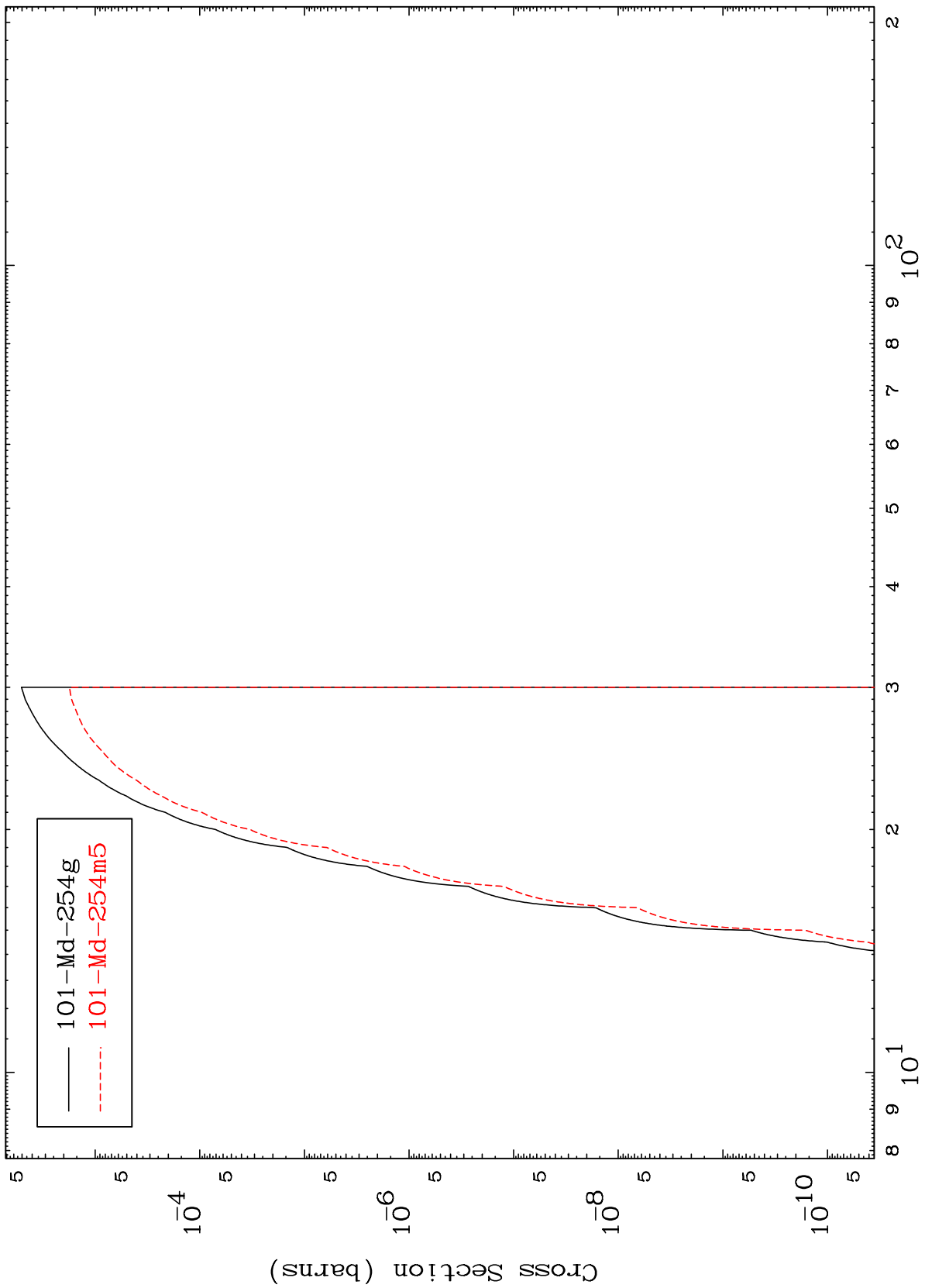


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(n,n') t

102-No-257

Radionuclide Production Cross Section



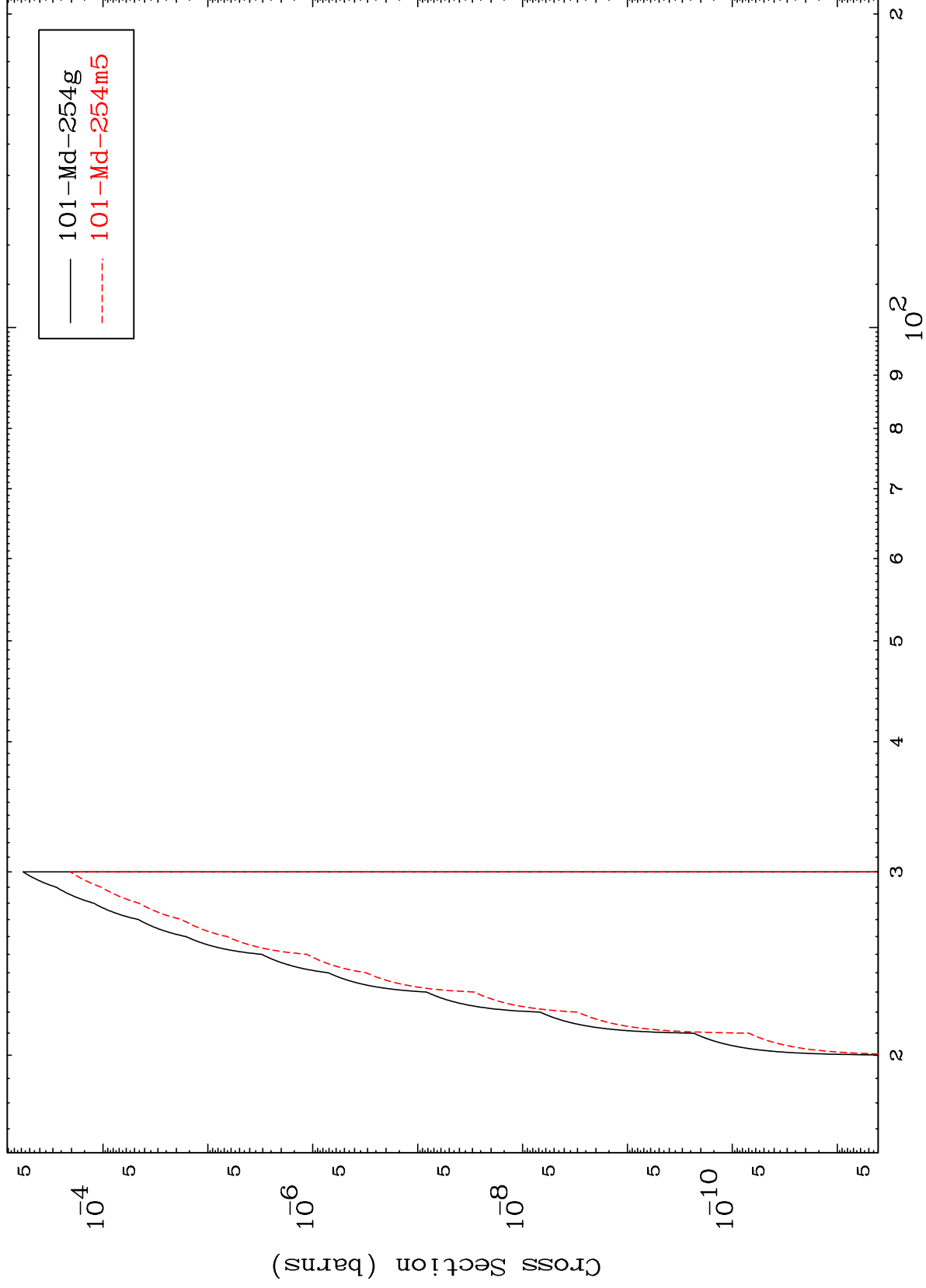
Legend:  
— 101-Md-254g  
- - - 101-Md-254m5

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Incident Energy (MeV)

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Radionuclide Production Cross Section



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(n,d) 2 $\alpha$

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Radionuclide Production Cross Section

