

Program EVALPLOT
(Version 2015-2)

by

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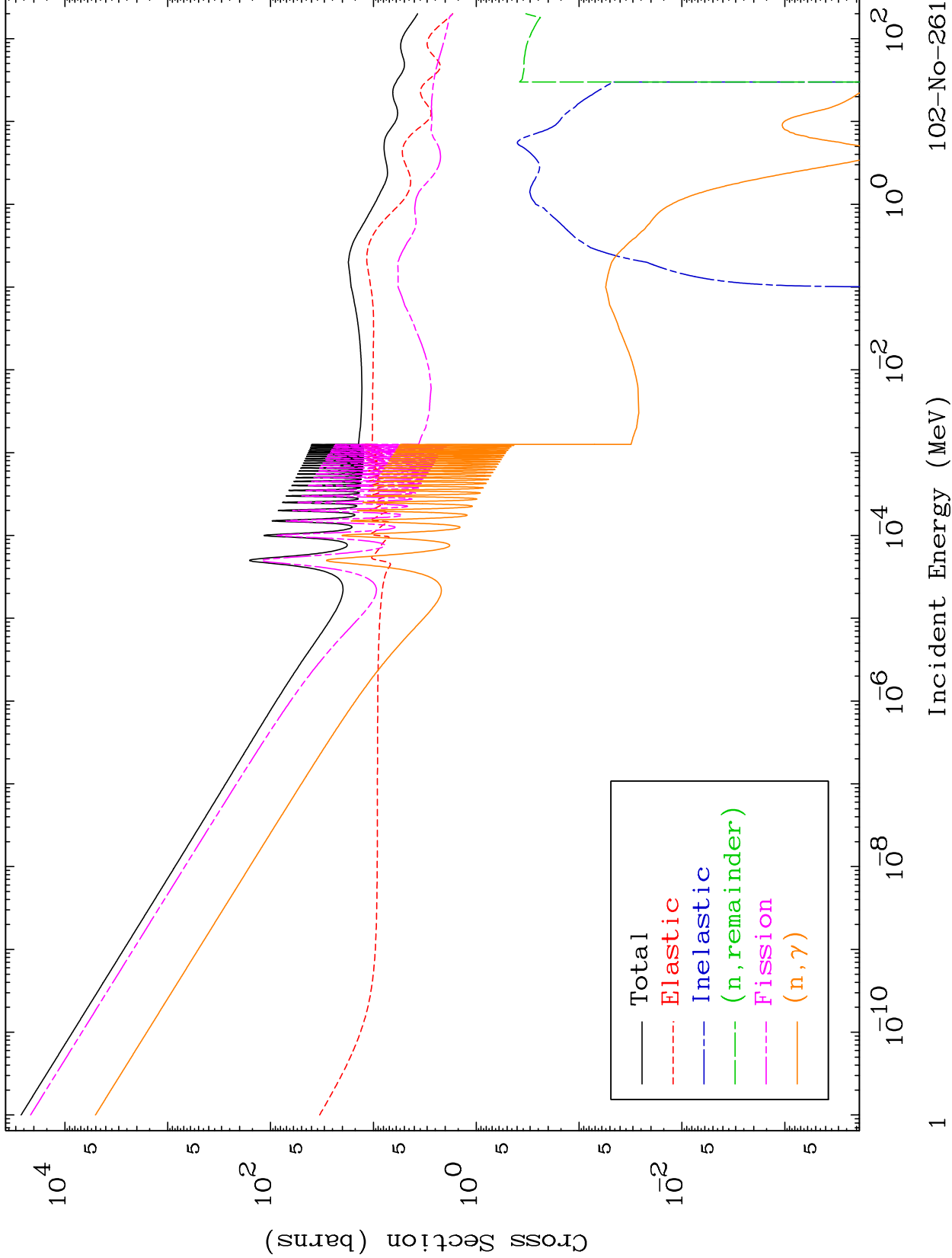
Press Mouse Button to Start

MAT 9979

Major

293 Kelvin Cross Sections

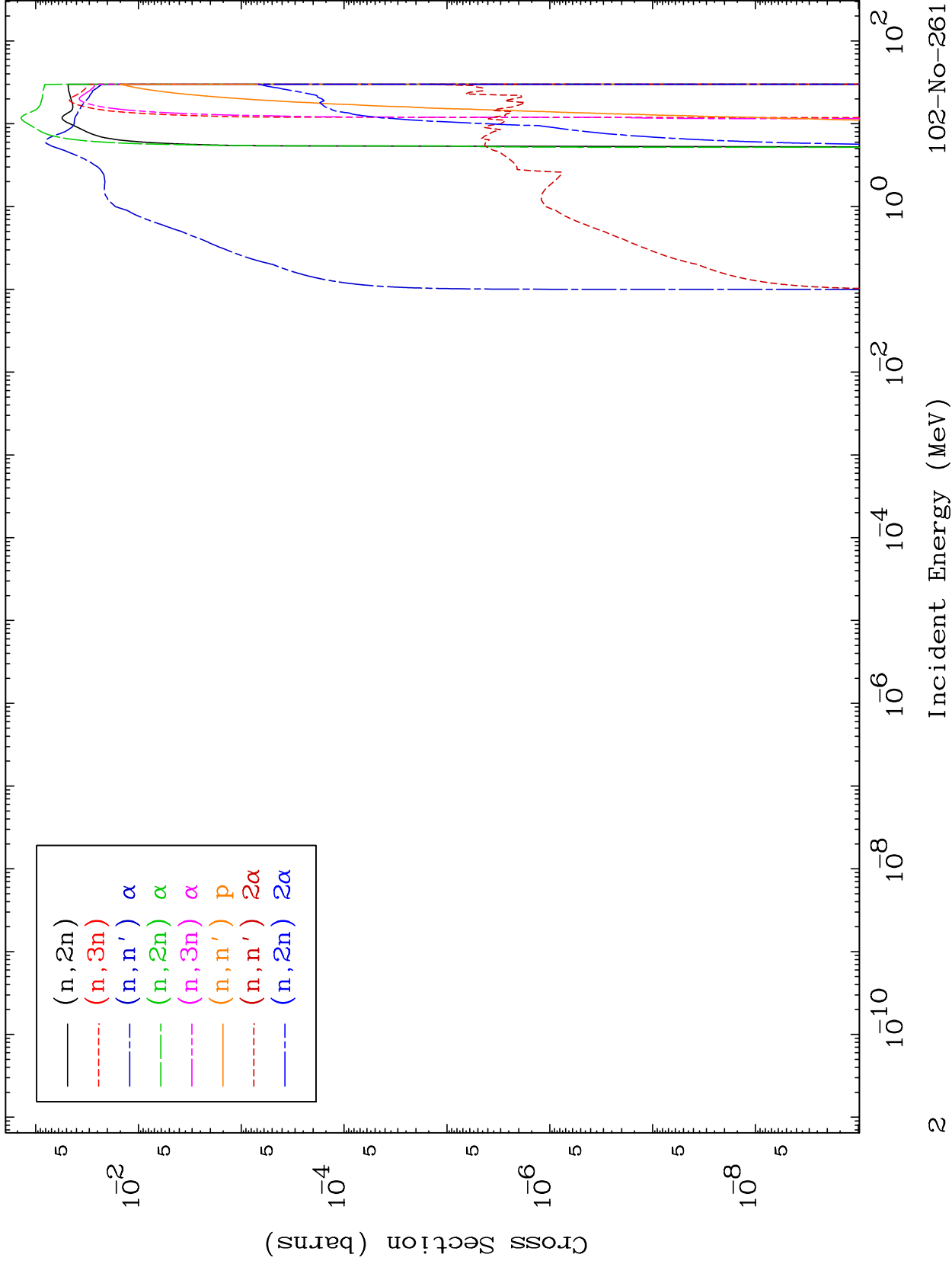
102-No-261



MAT 9979

Neutron Production
293 Kelvin Cross Sections

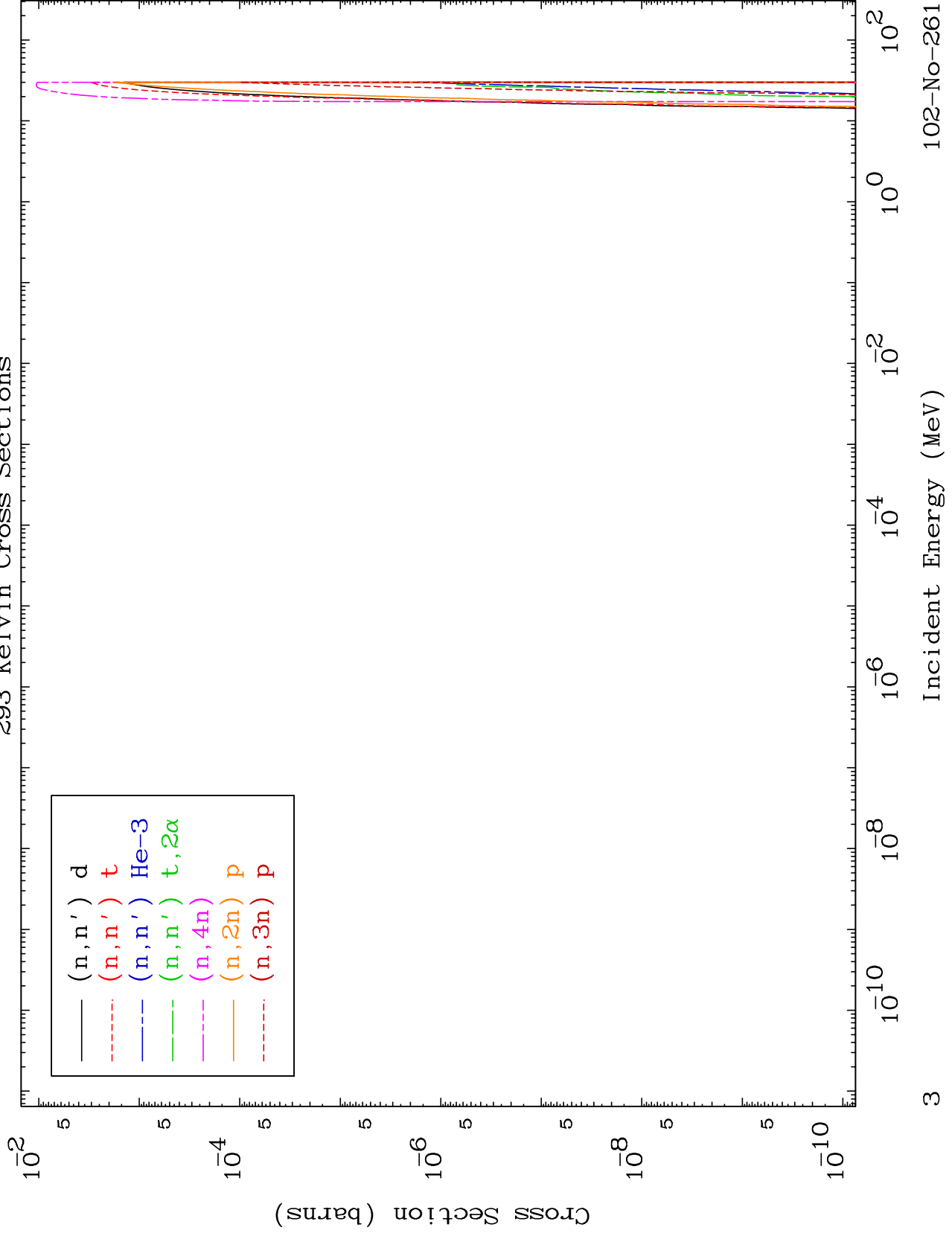
102-No-261



MAT 9979

Neutron Production
293 Kelvin Cross Sections

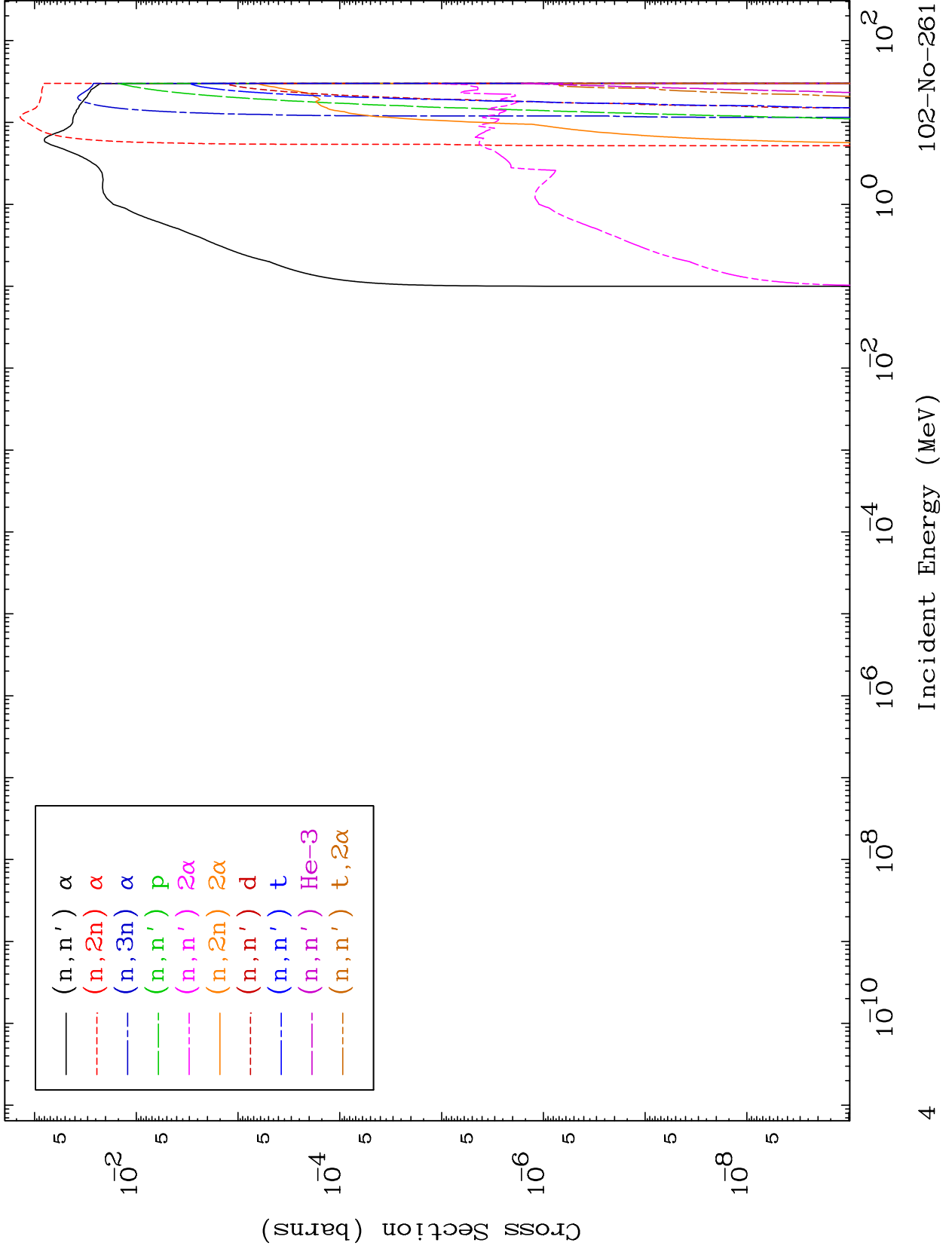
102-No-261



MAT 9979

Charged Particle
293 Kelvin Cross Sections

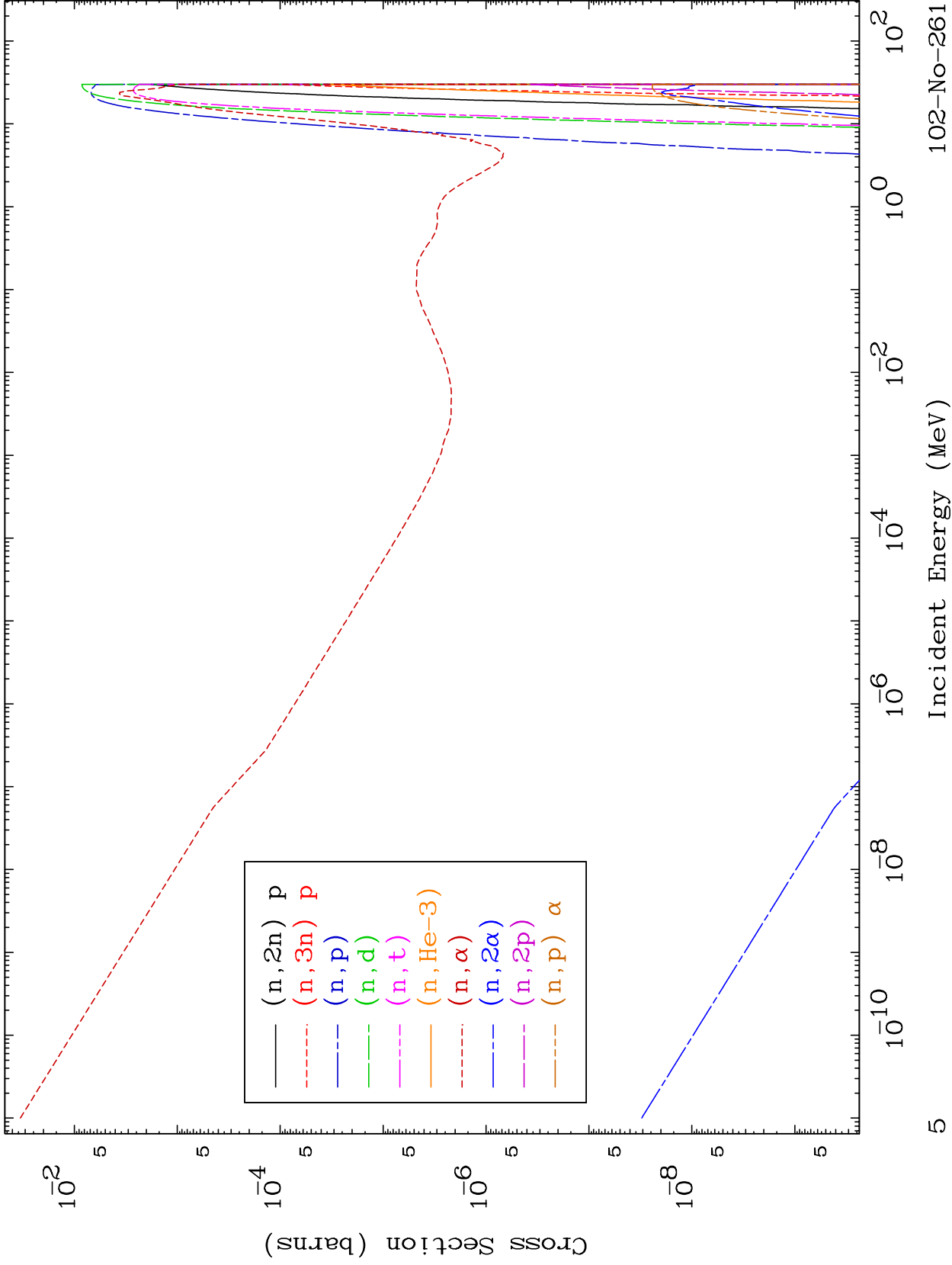
102-No-261

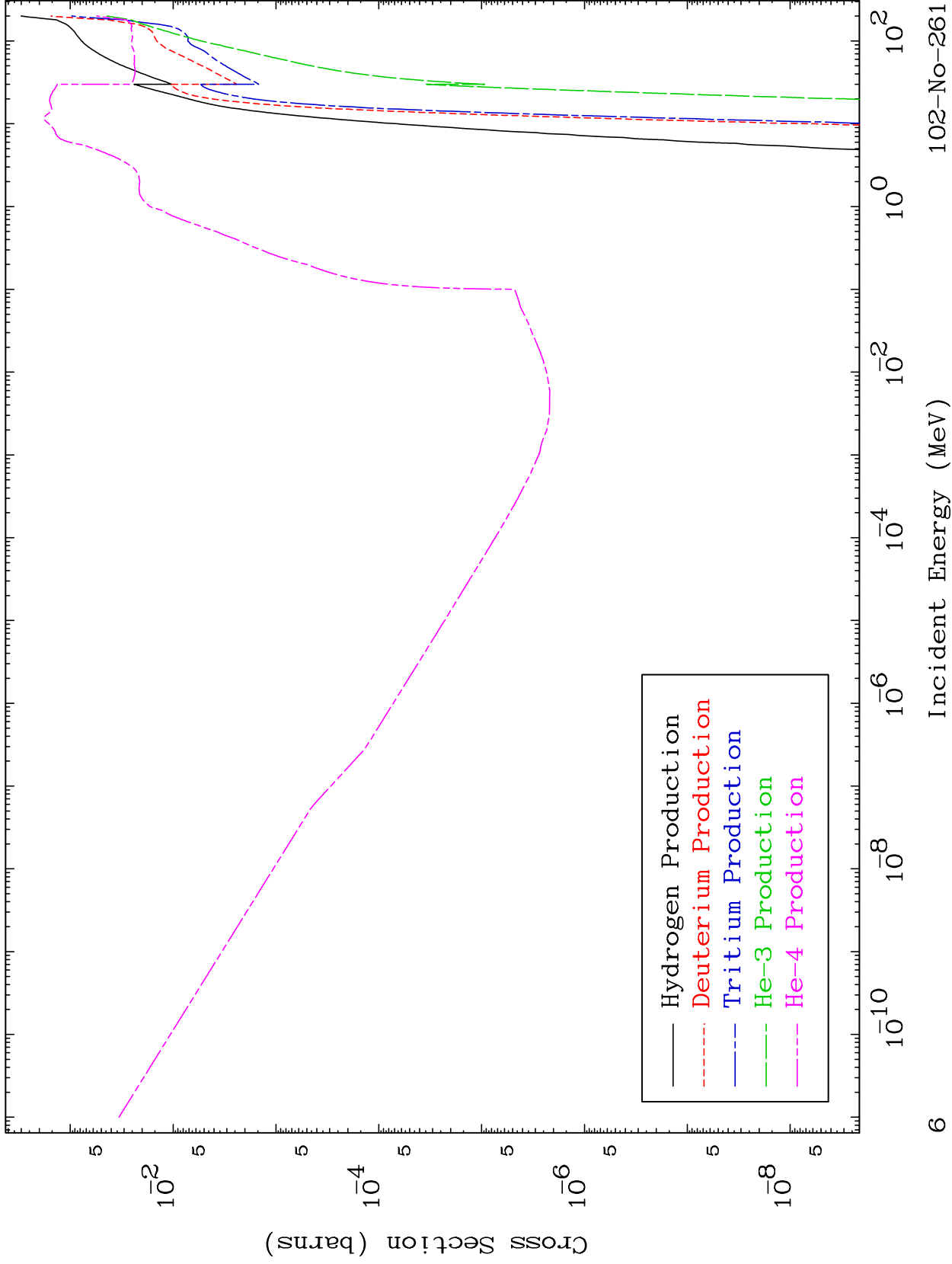


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Charged Particle
293 Kelvin Cross Sections

102-No-261

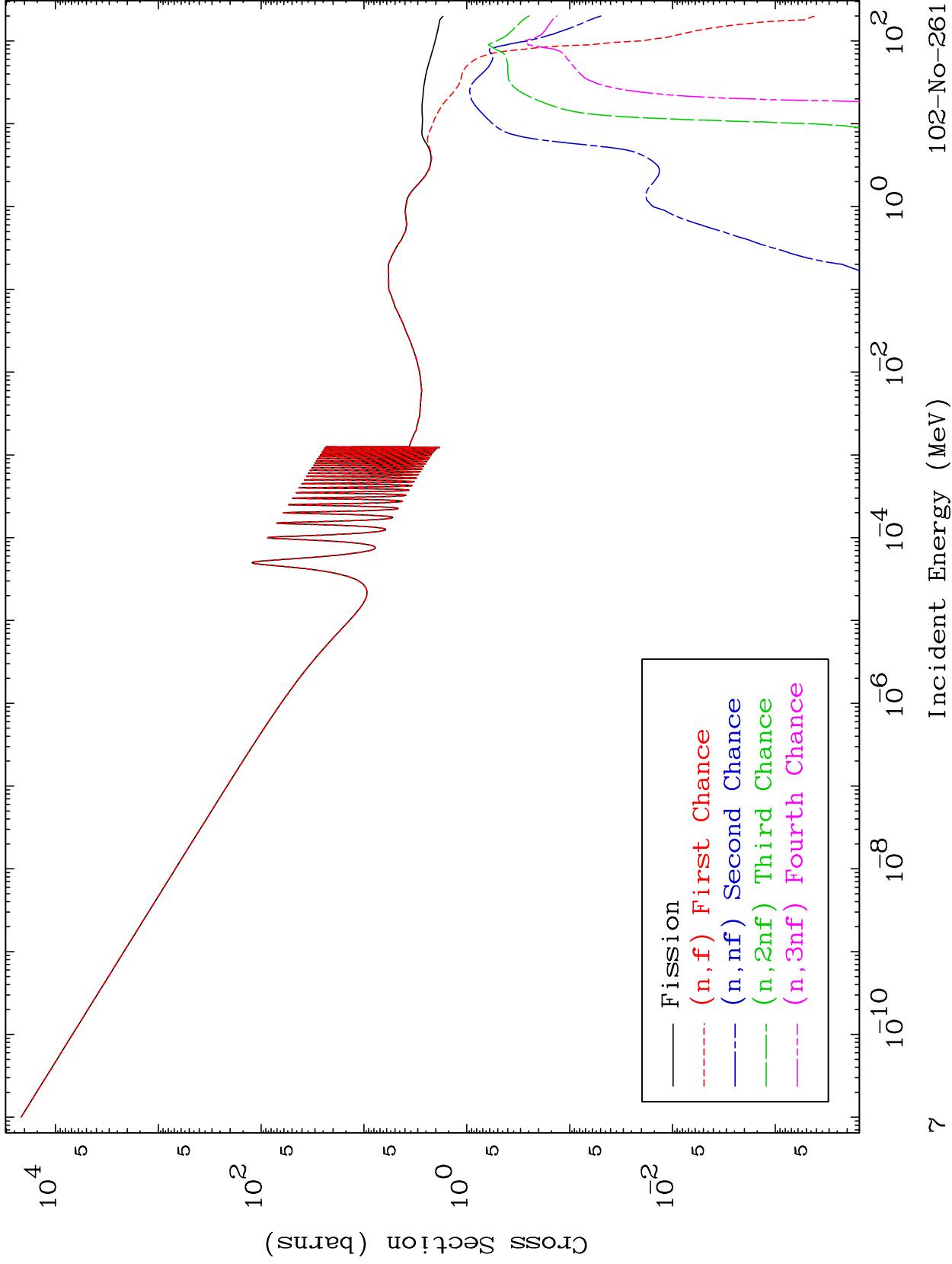




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Fission
293 Kelvin Cross Sections

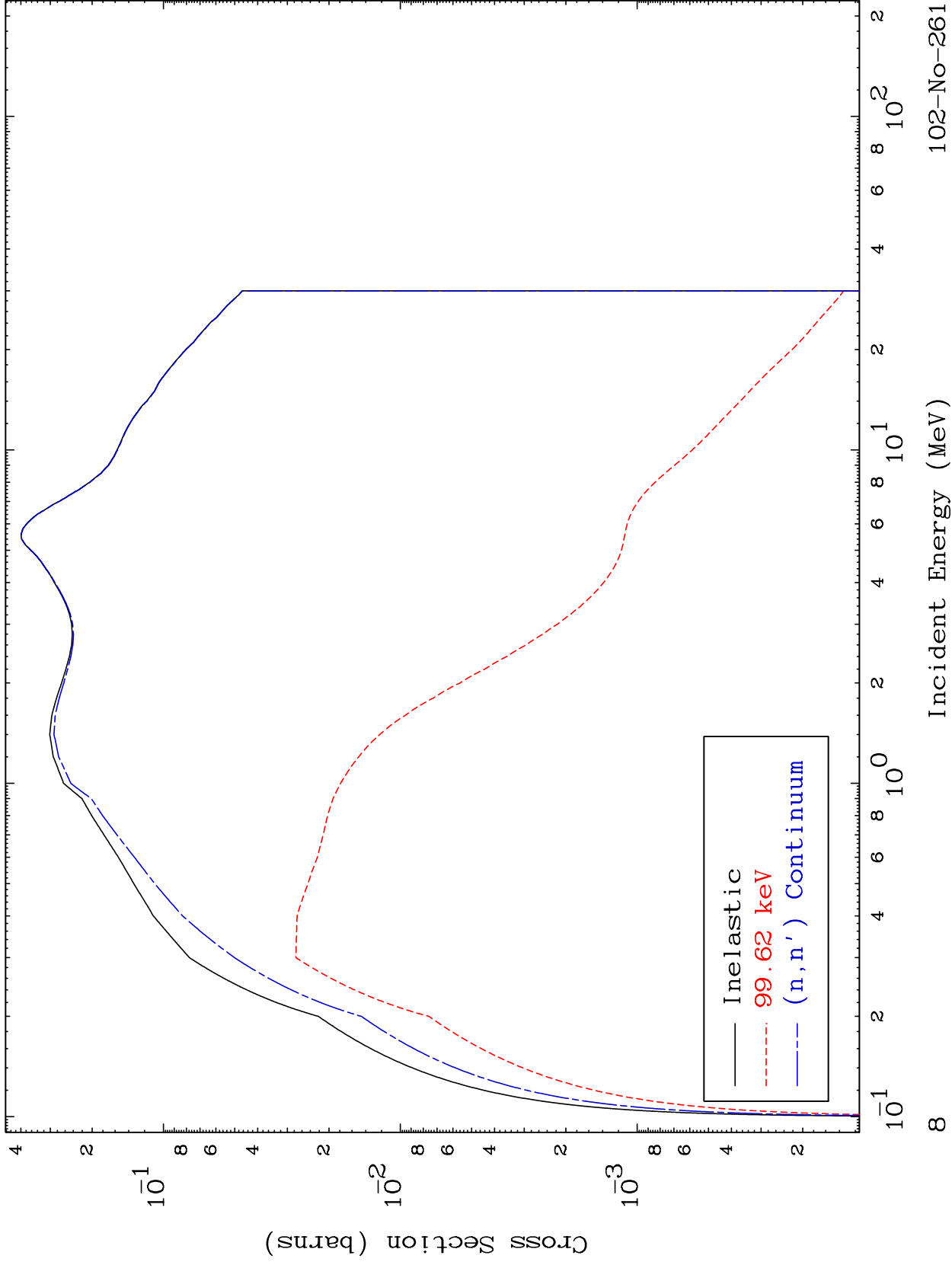
102-No-261



MAT 9979

(n,n') Level
293 Kelvin Cross Sections

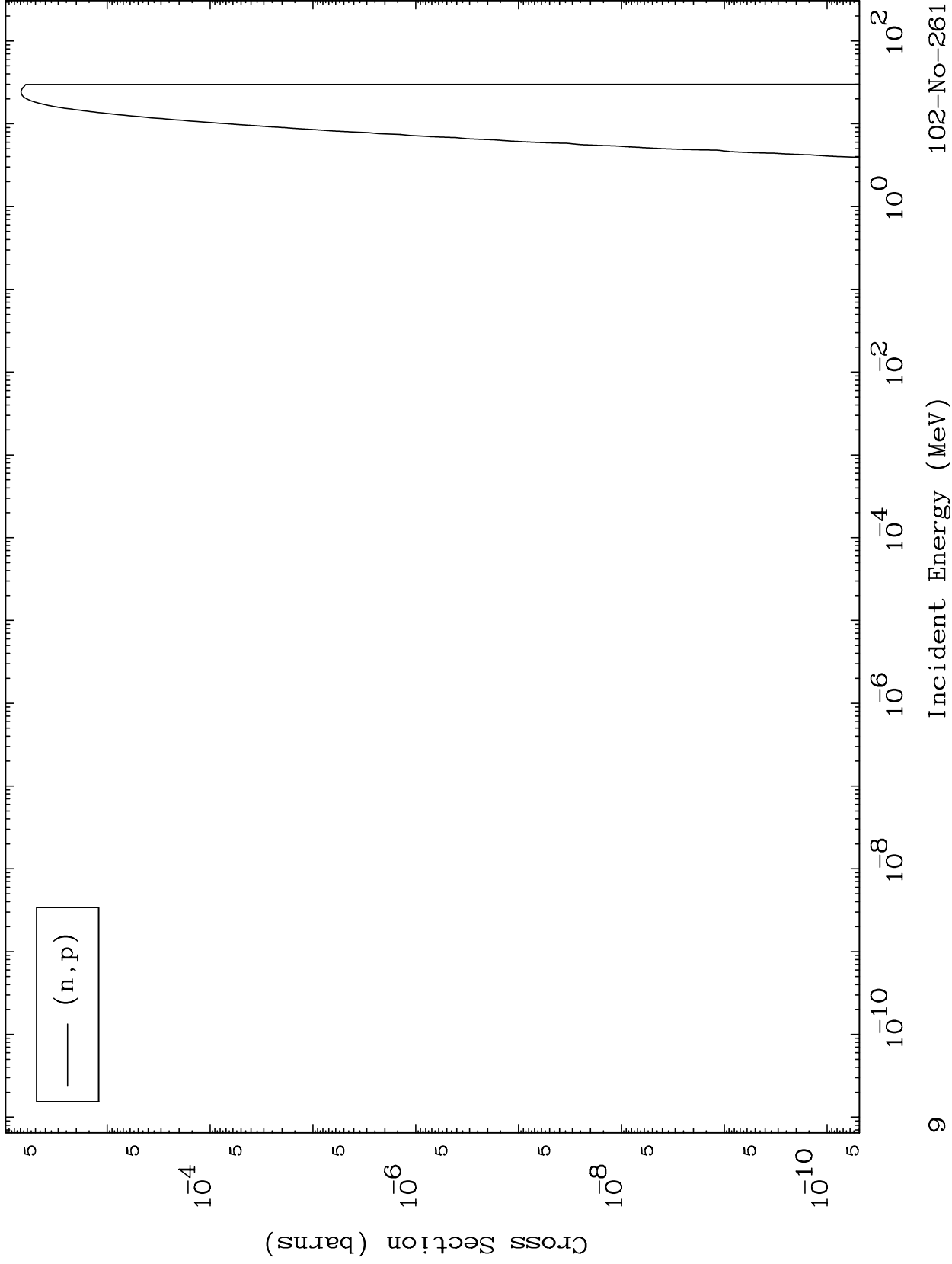
102-No-261



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(n,p) Levels
293 Kelvin Cross Sections

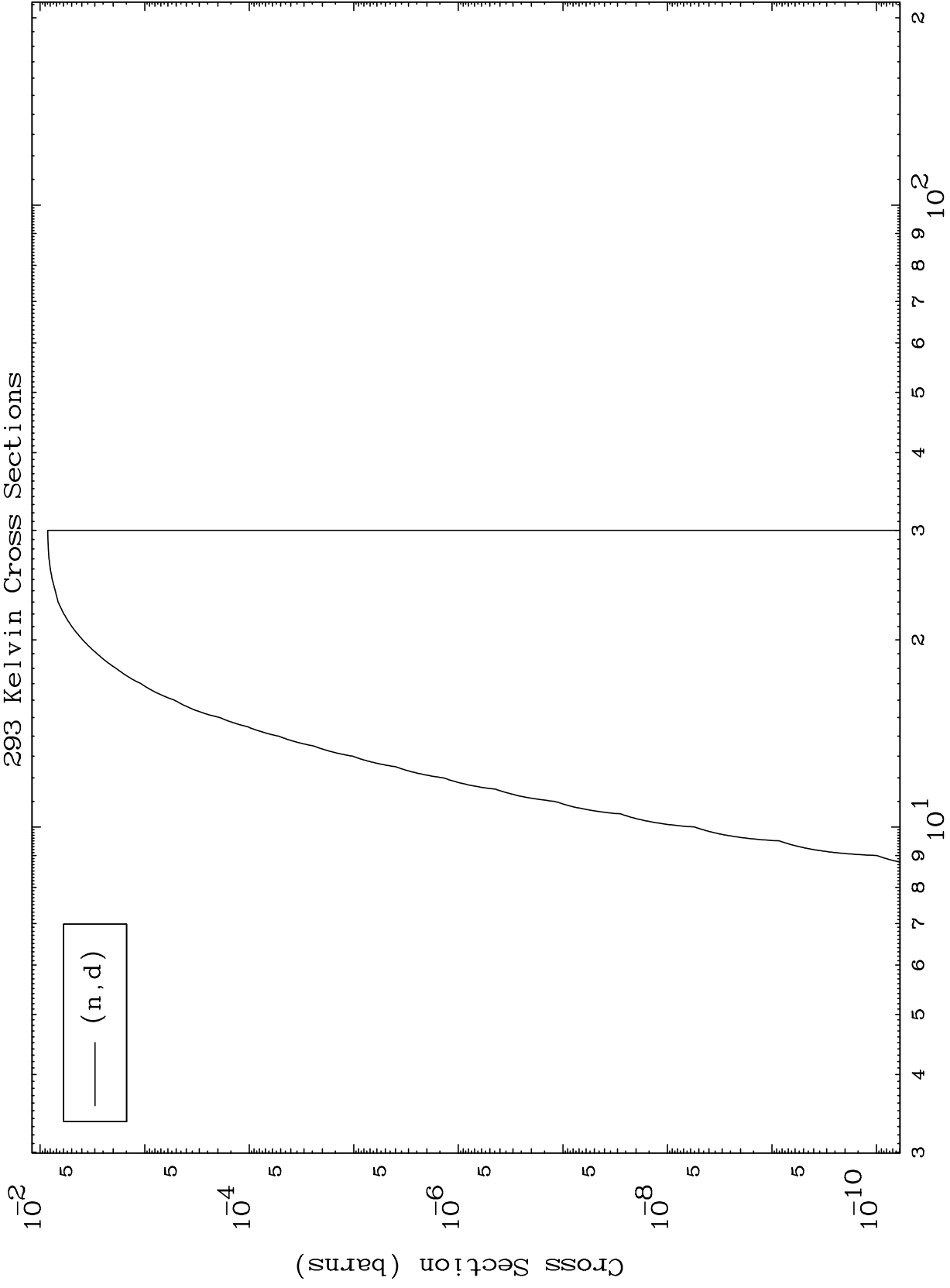
102-No-261



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(n,d) Levels
293 Kelvin Cross Sections

102-No-261



102-No-261

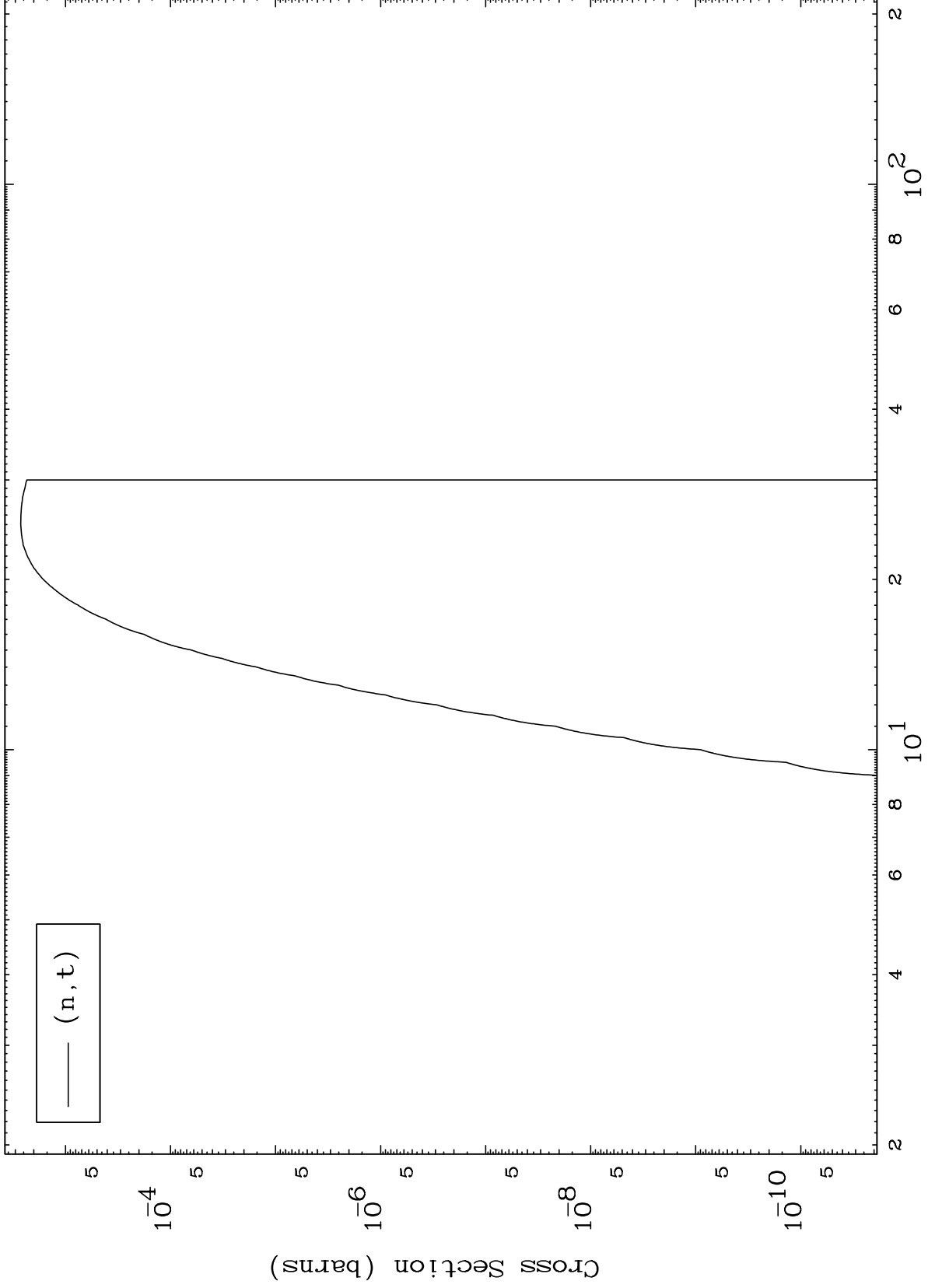
Incident Energy (MeV)

10

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(n,t) Levels
293 Kelvin Cross Sections

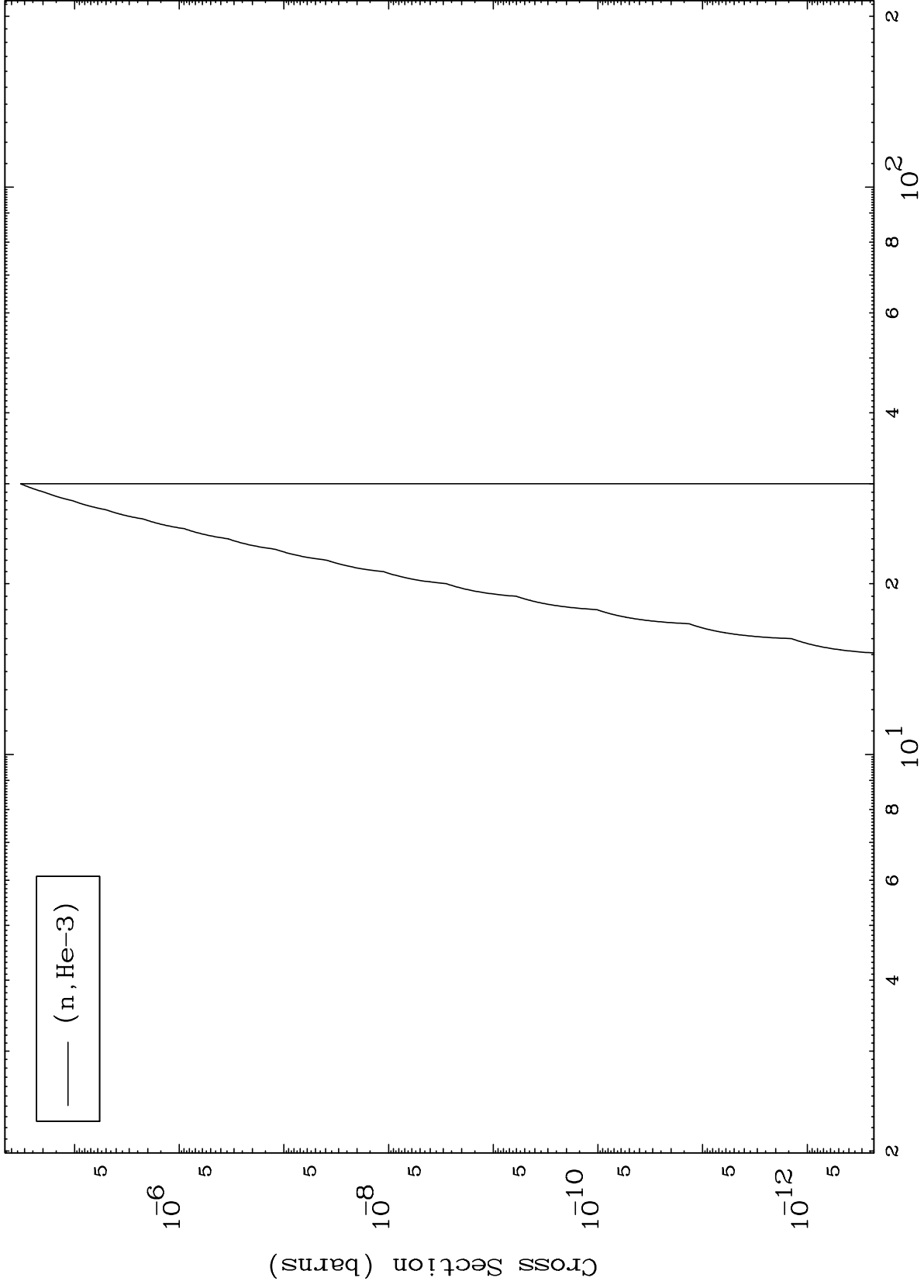
102-No-261



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(n,He3) Levels
293 Kelvin Cross Sections

102-No-261



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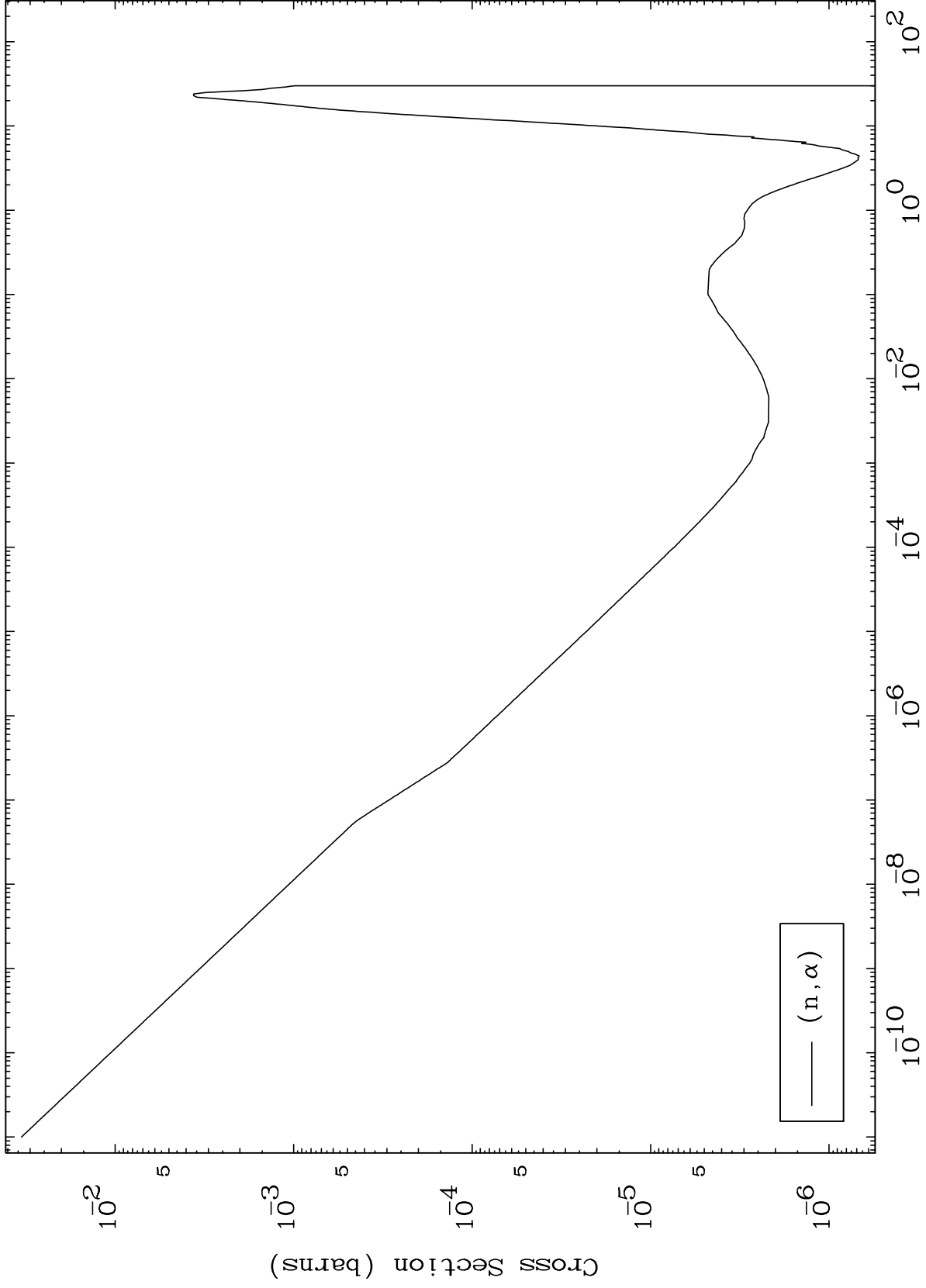
Incident Energy (MeV)

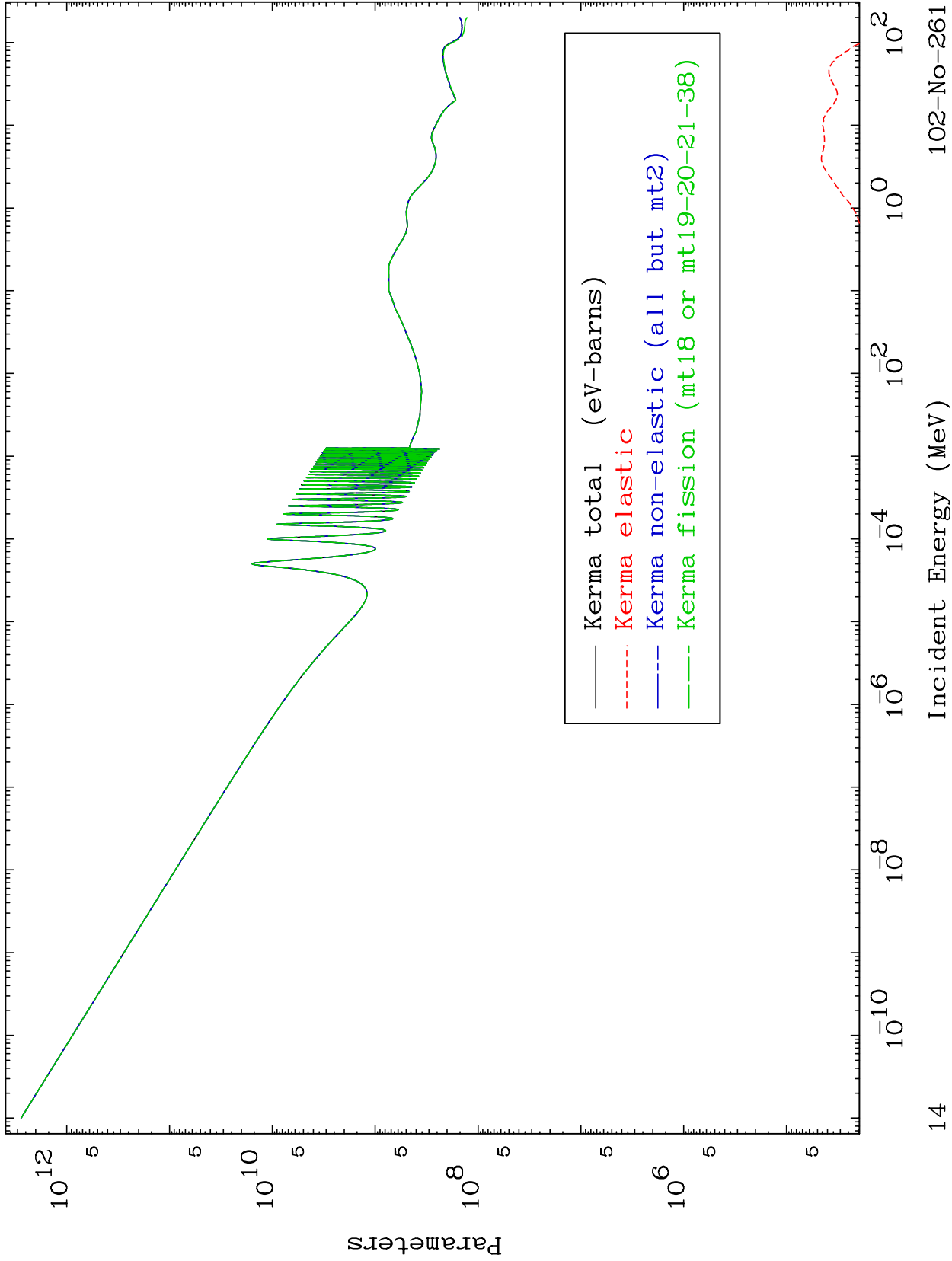
102-No-261

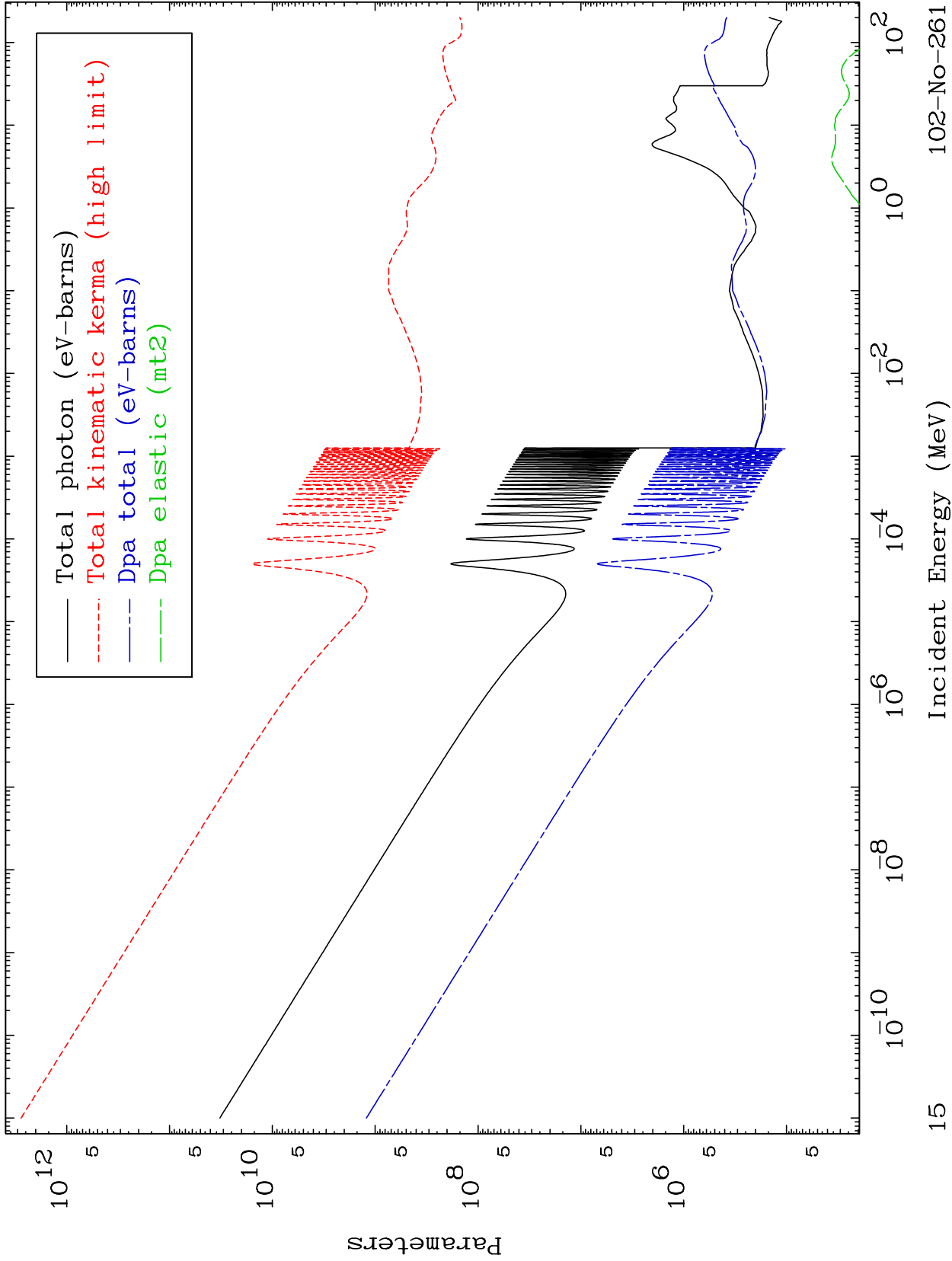
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(n,α) Levels
293 Kelvin Cross Sections

102-No-261



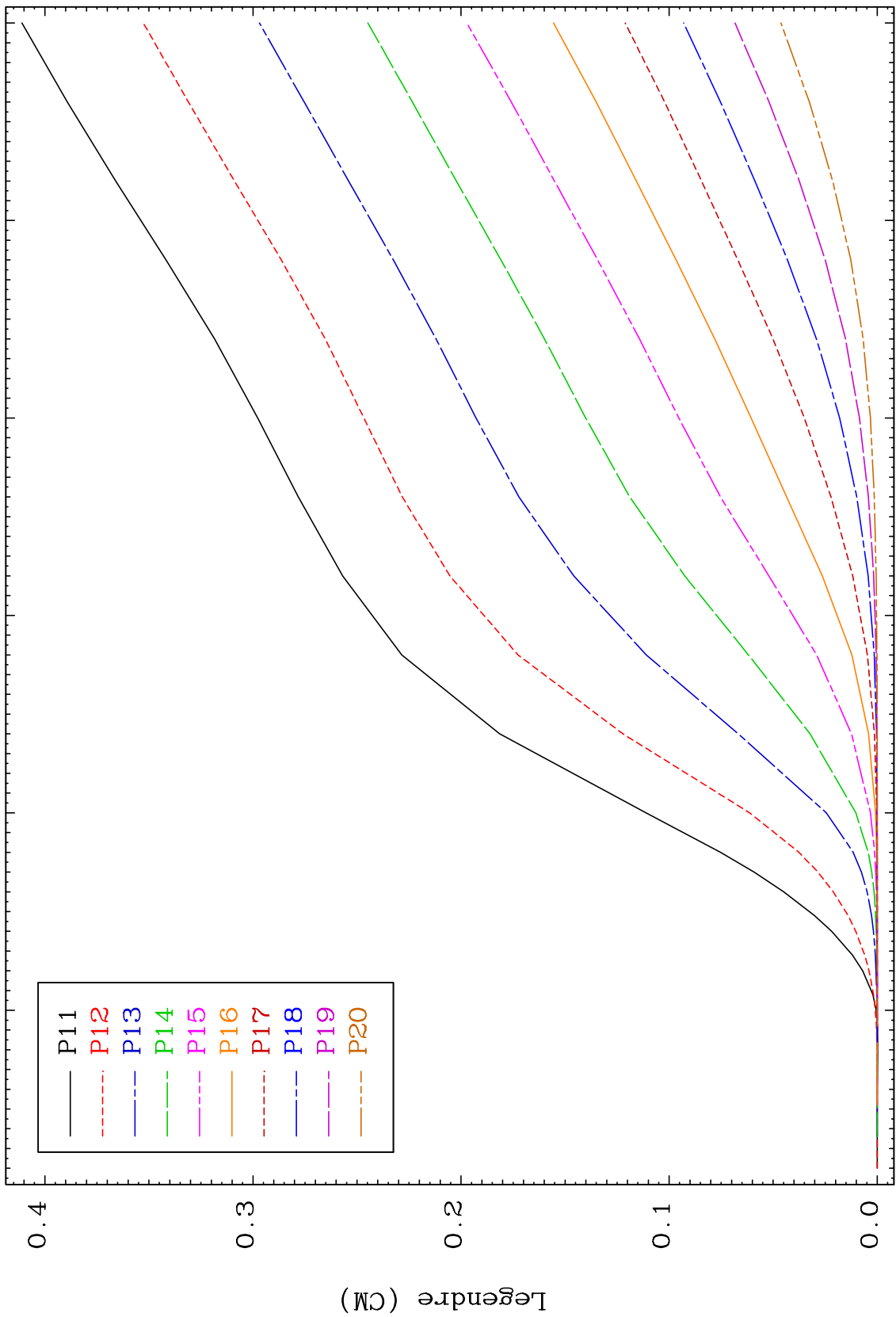




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Elastic Legendre Coefficients

102-No-261



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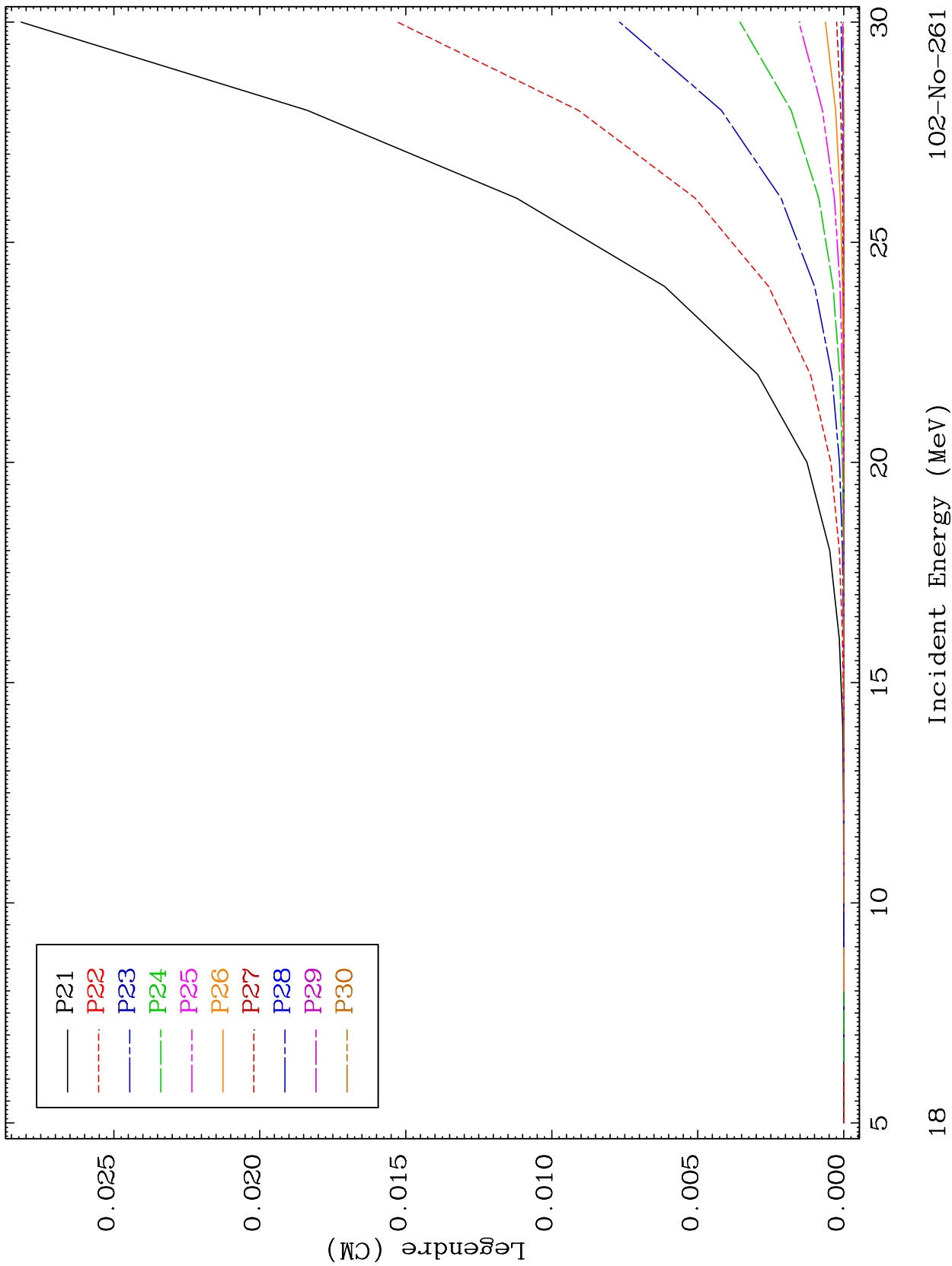
Incident Energy (MeV)

102-No-261

MAT 9979

Elastic Legendre Coefficients

102-No-261



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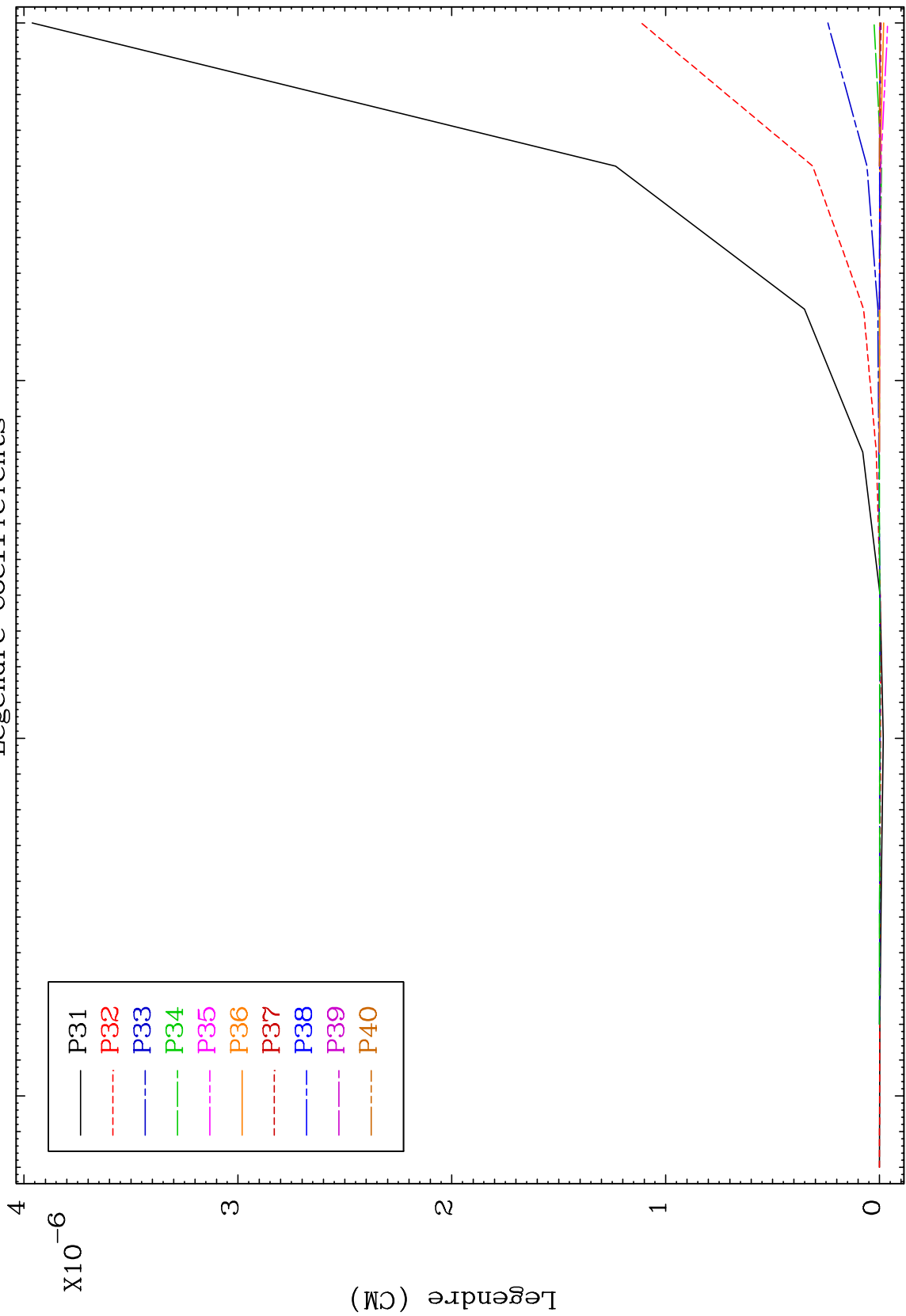
Incident Energy (MeV)

102-No-261

MAT 9979

Elastic
Legendre Coefficients

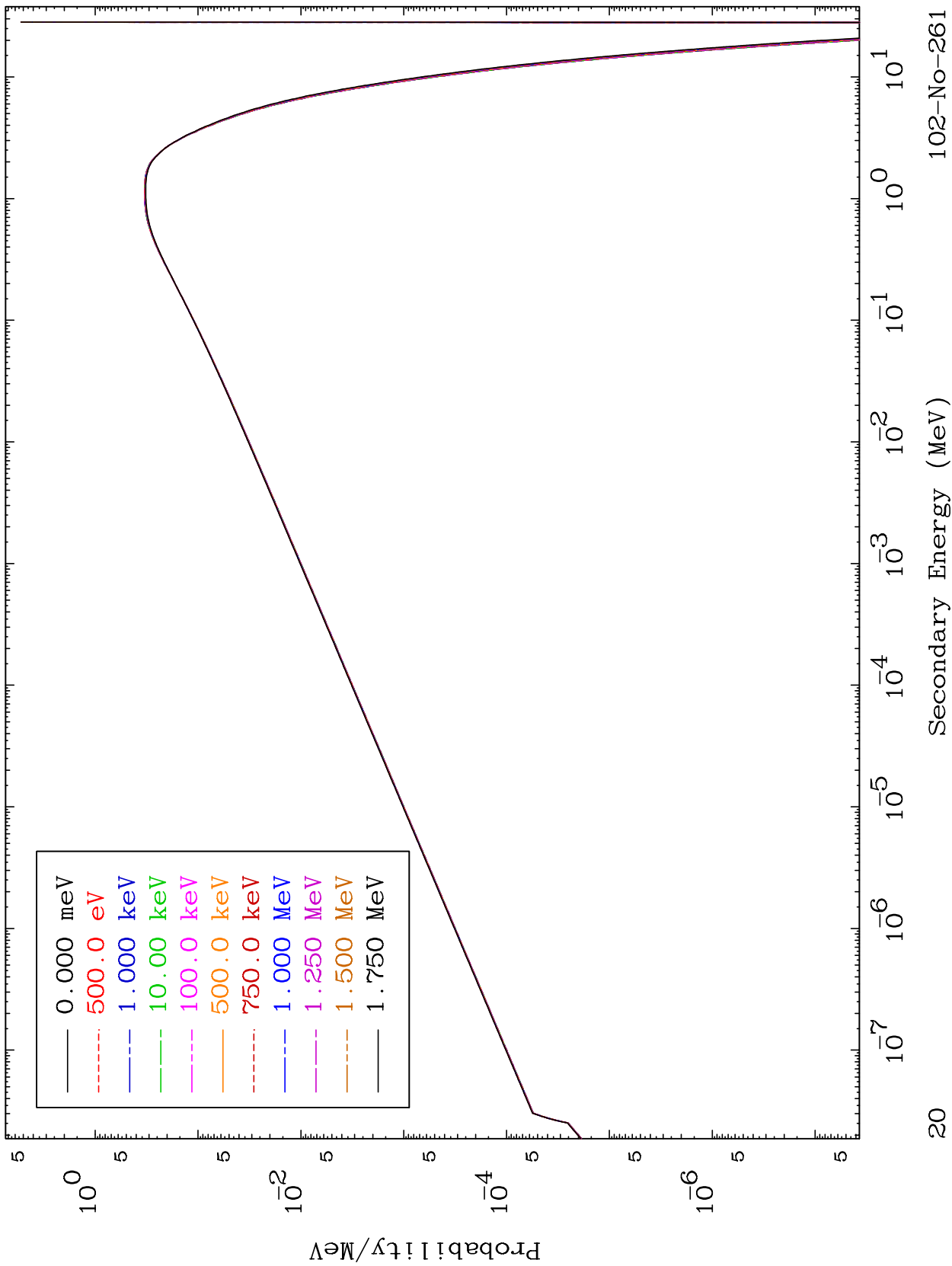
102-No-261



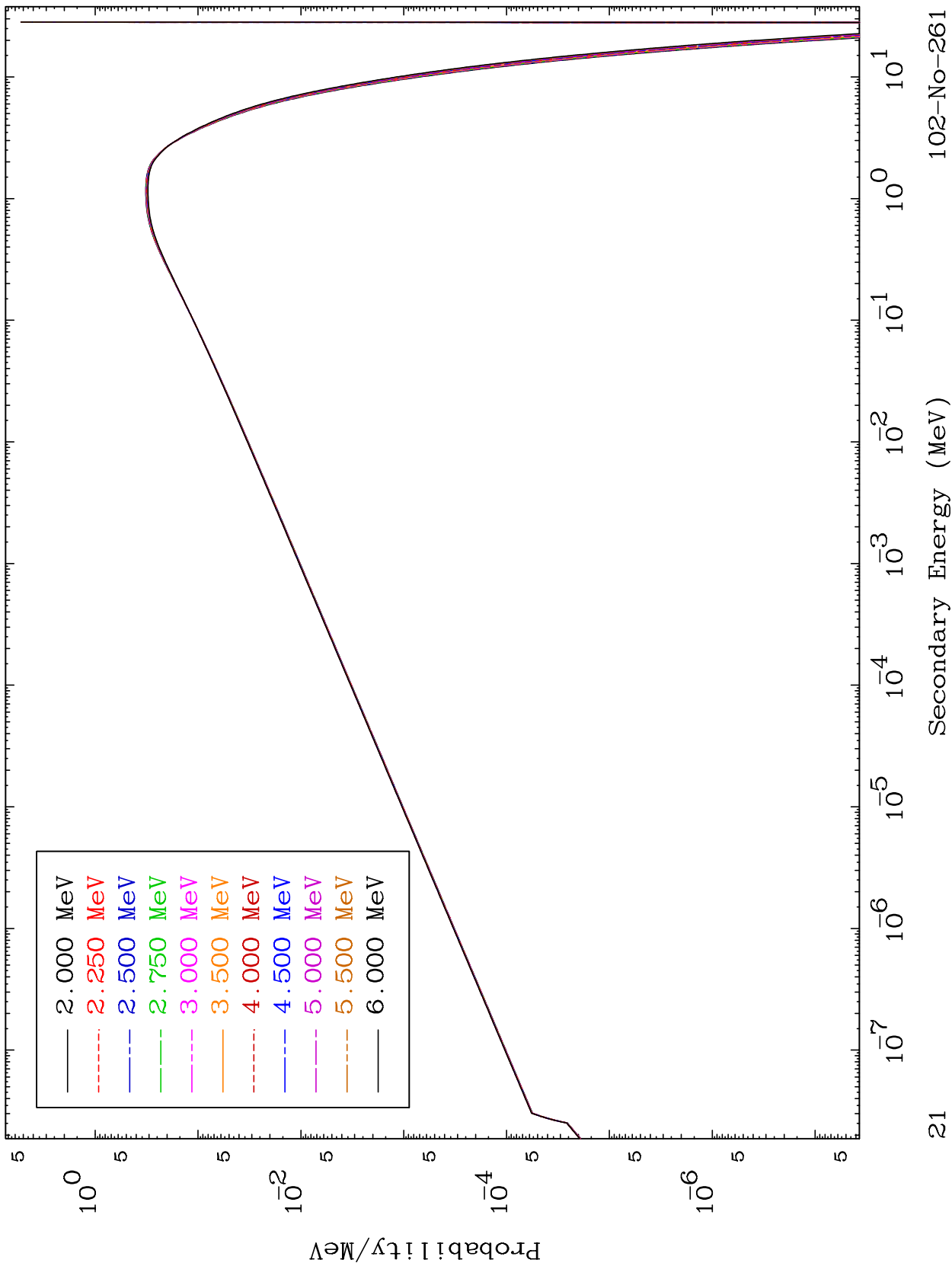
19

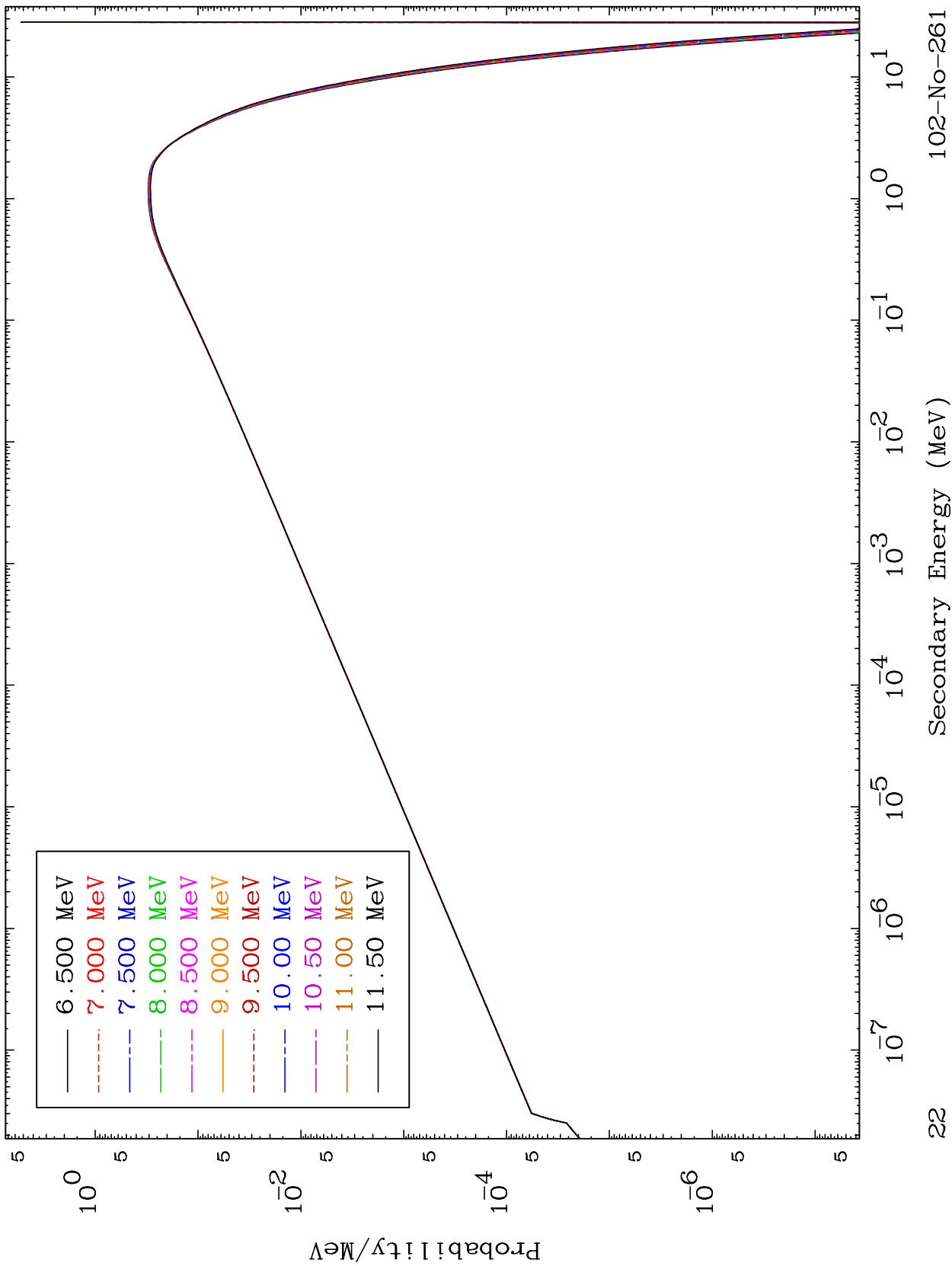
Incident Energy (MeV)

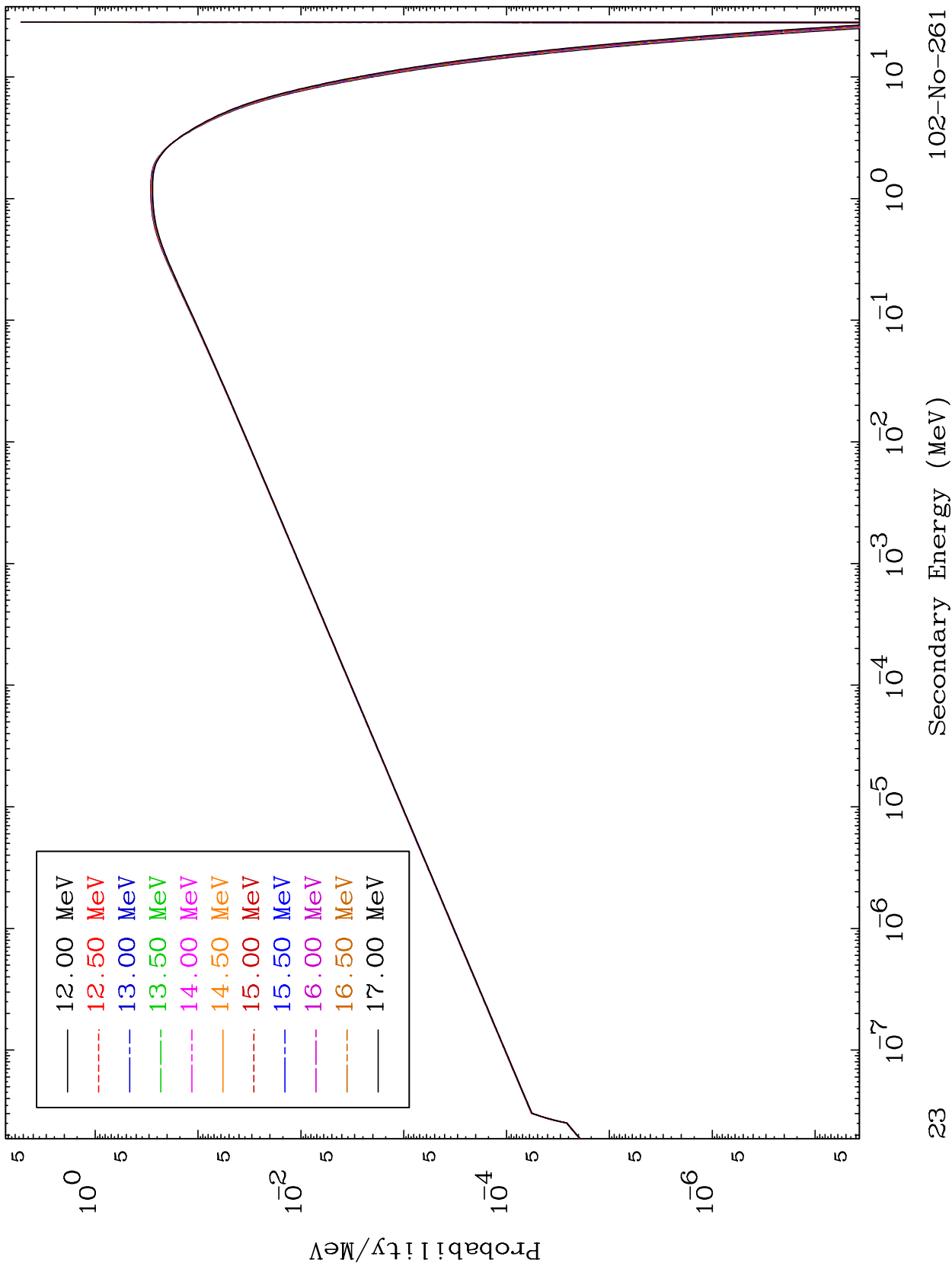
102-No-261



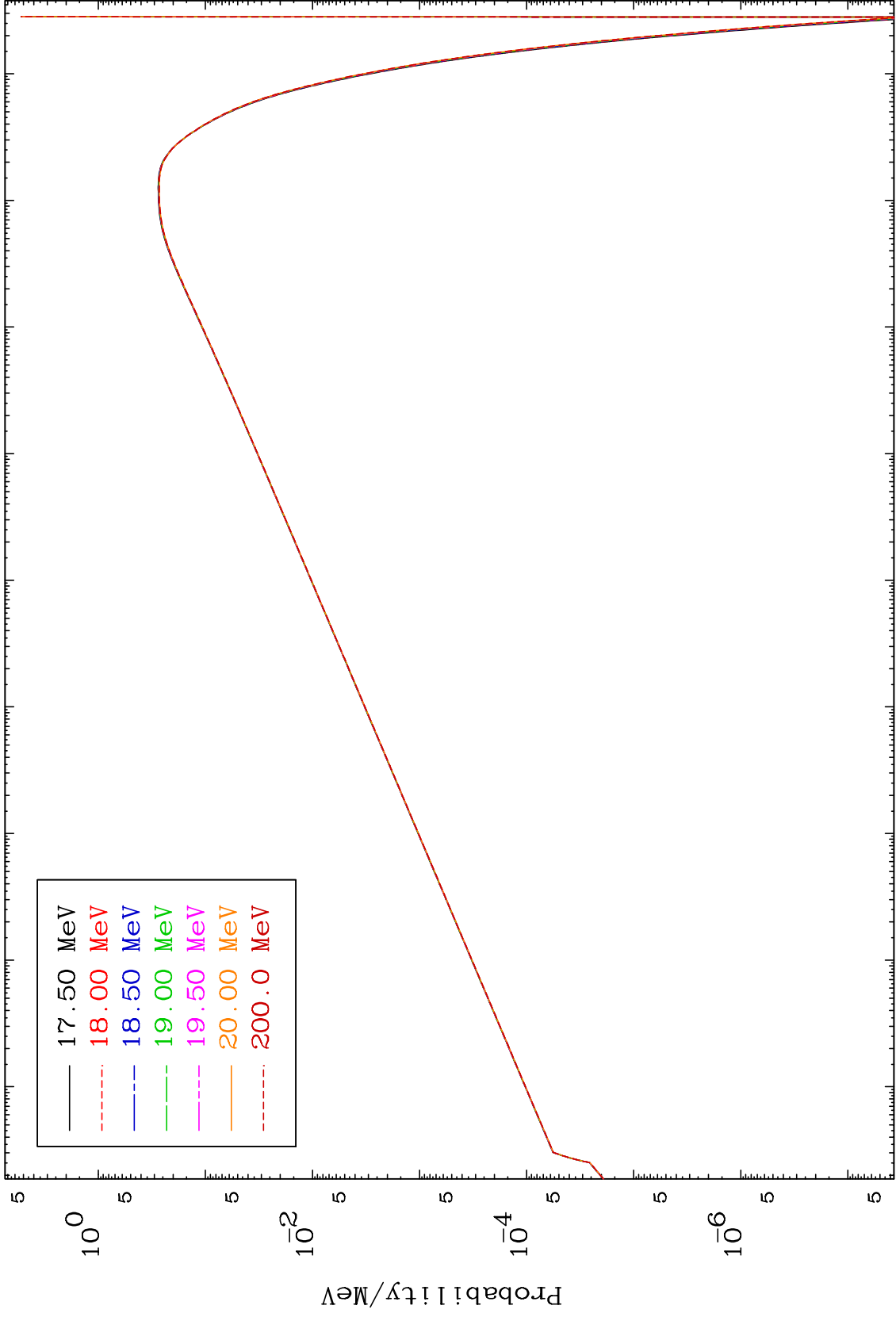
Fission
Energy Distribution



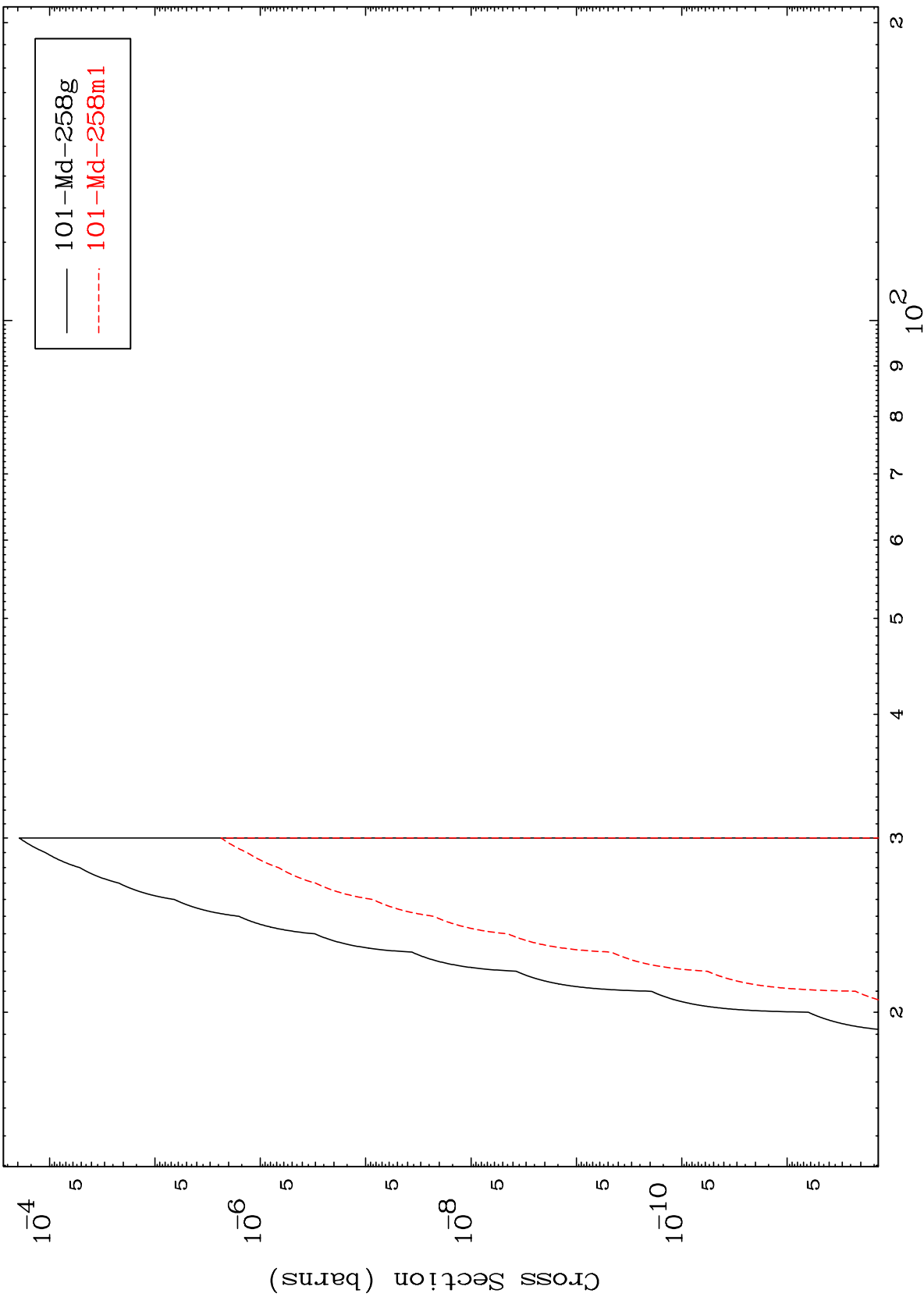




Fission
Energy Distribution



Radionuclide Production Cross Section



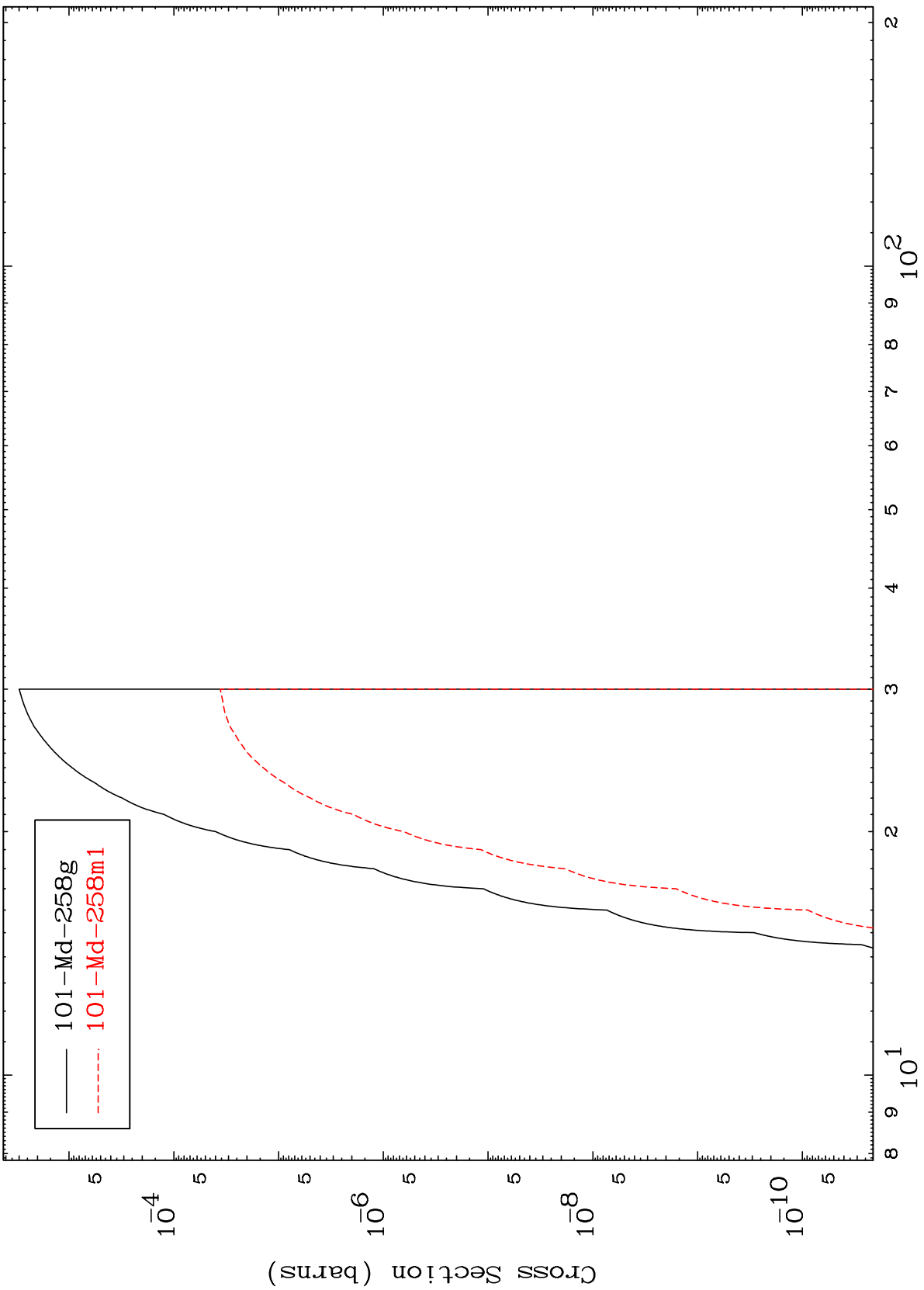
101-Md-258g
101-Md-258m1

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(n,n') t

102-No-261

Radionuclide Production Cross Section

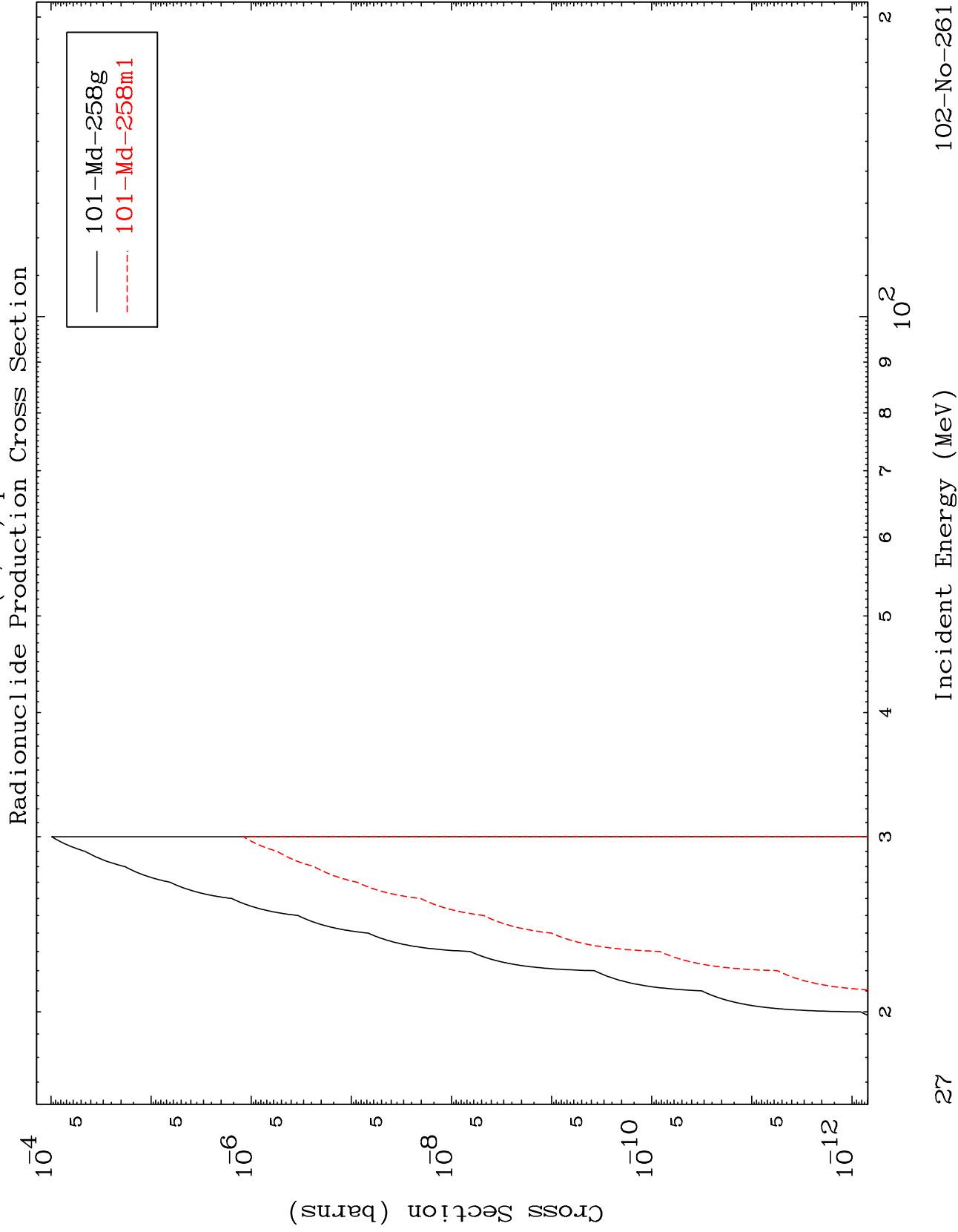


101-Md-258g
101-Md-258m1

26

Incident Energy (MeV)

102-No-261

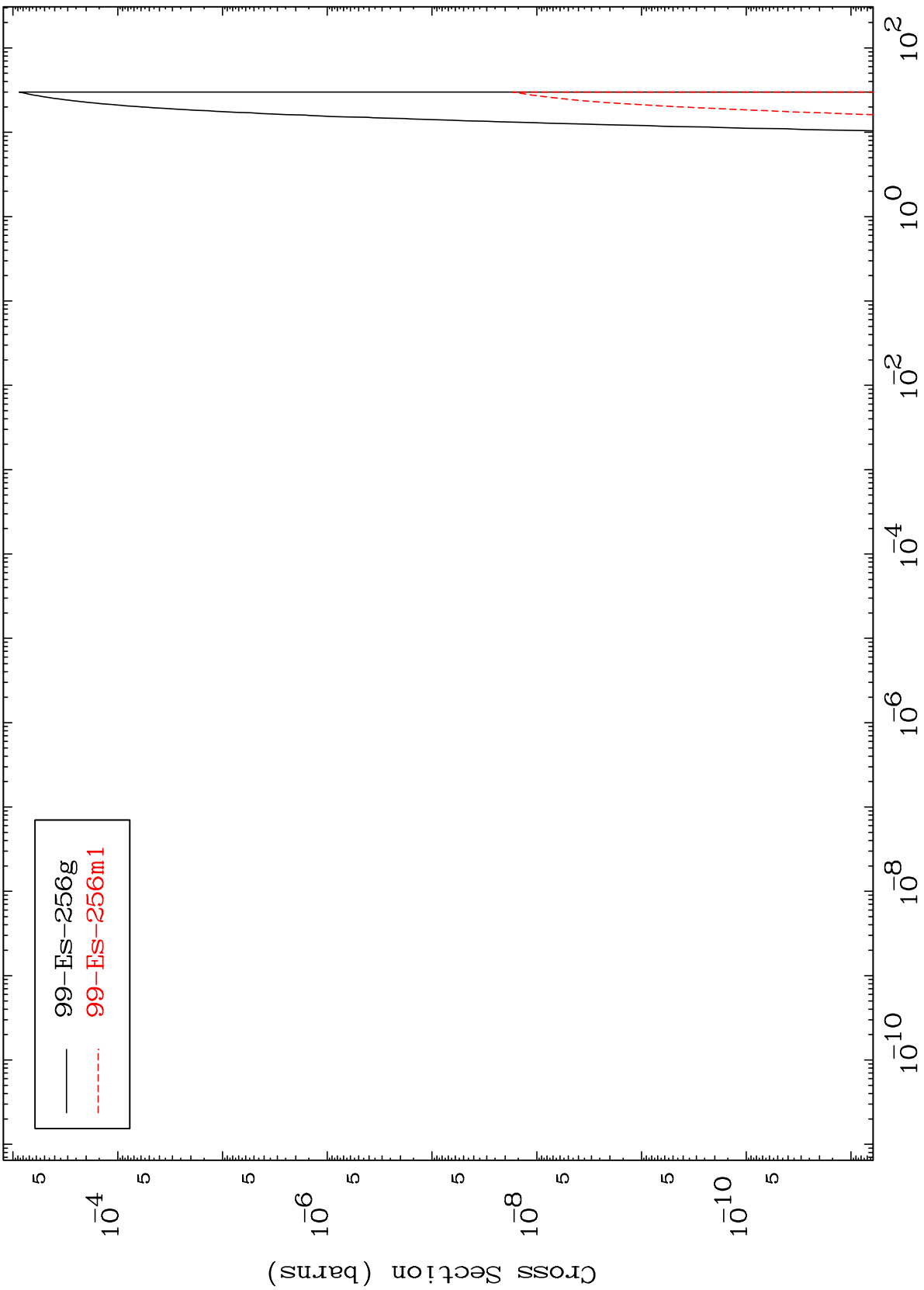


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(n,n') p α

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Radionuclide Production Cross Section



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Incident Energy (MeV)

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(n,d) α

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Radionuclide Production Cross Section

