

Program EVALPLOT  
(Version 2015-2)

by

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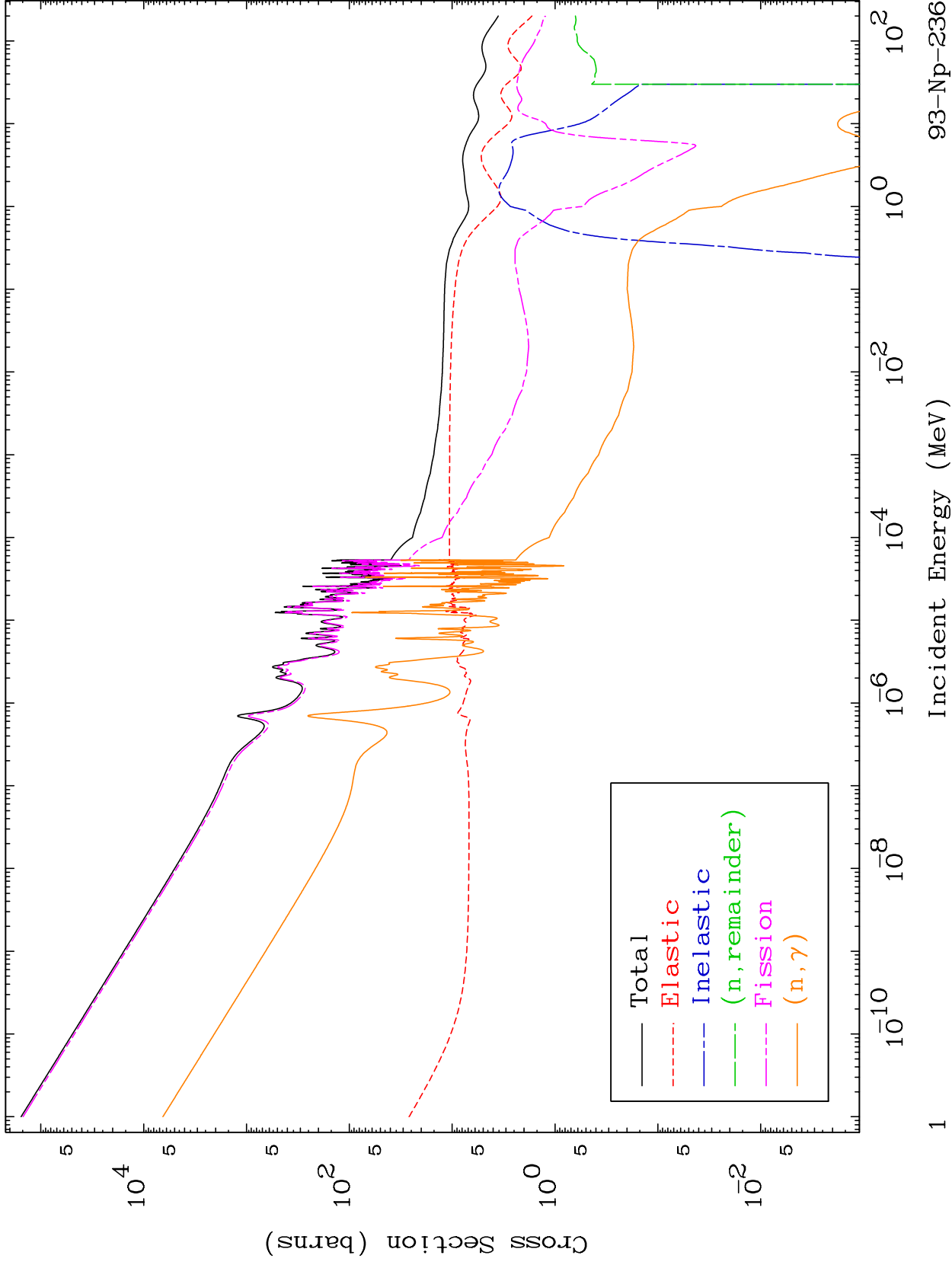
E.Mail:redcullen1@comcast.net  
Web:home.comcast.net/~redcullen1

Press Mouse Button to Start

MAT 9343

Major  
293 Kelvin Cross Sections

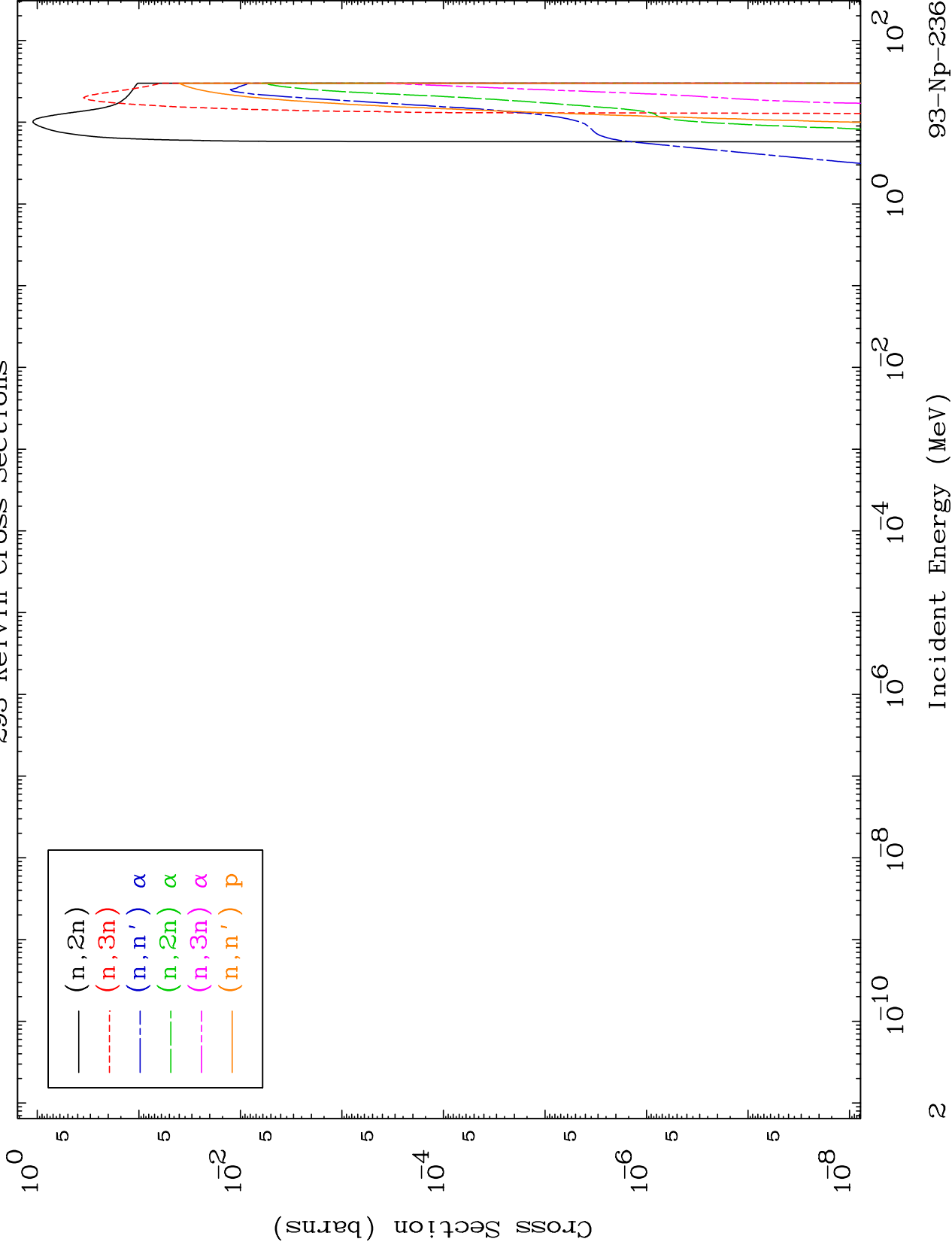
93-Np-236



MAT 9343

Neutron Production  
293 Kelvin Cross Sections

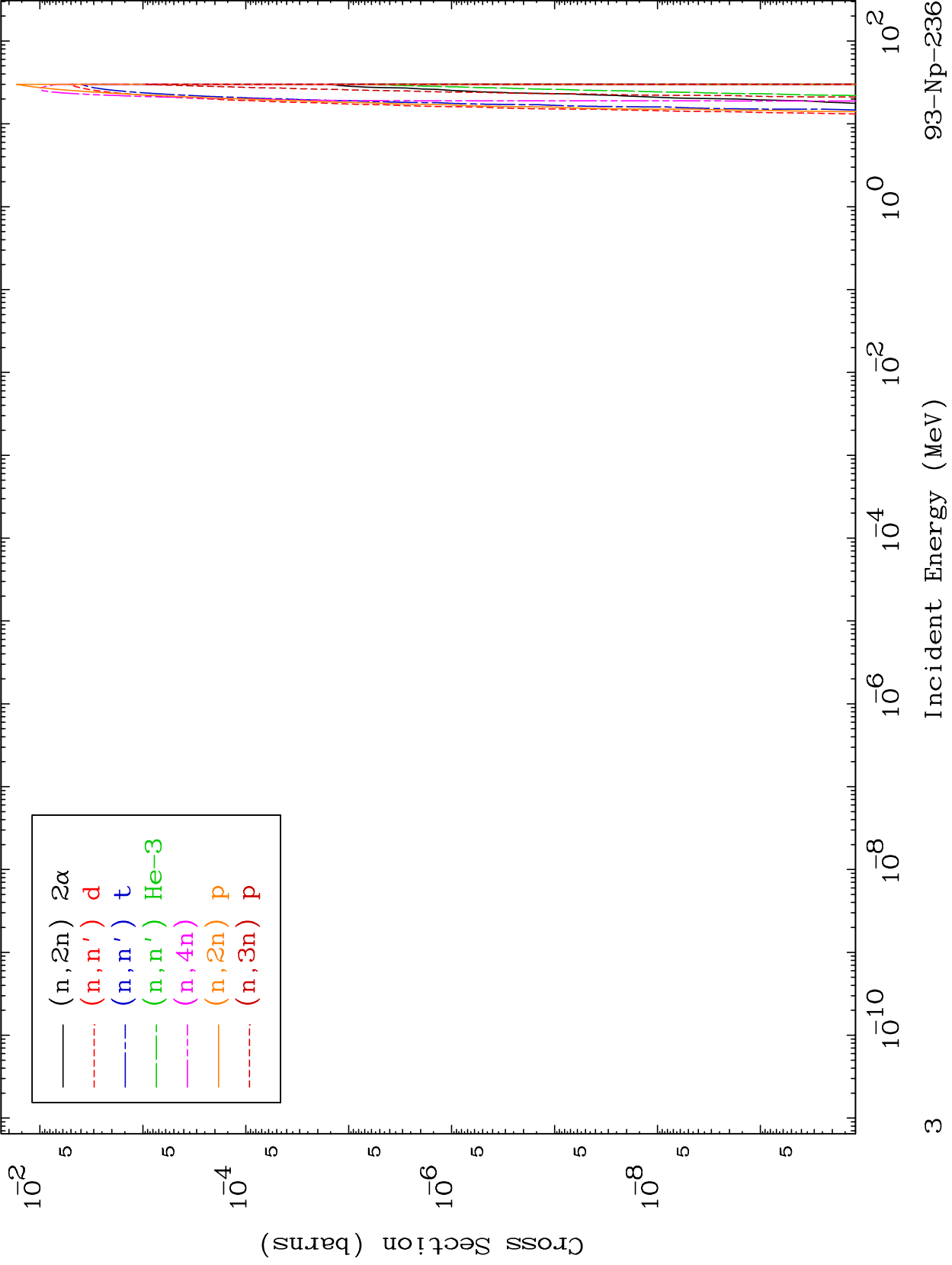
93-Np-236



MAT 9343

Neutron Production  
293 Kelvin Cross Sections

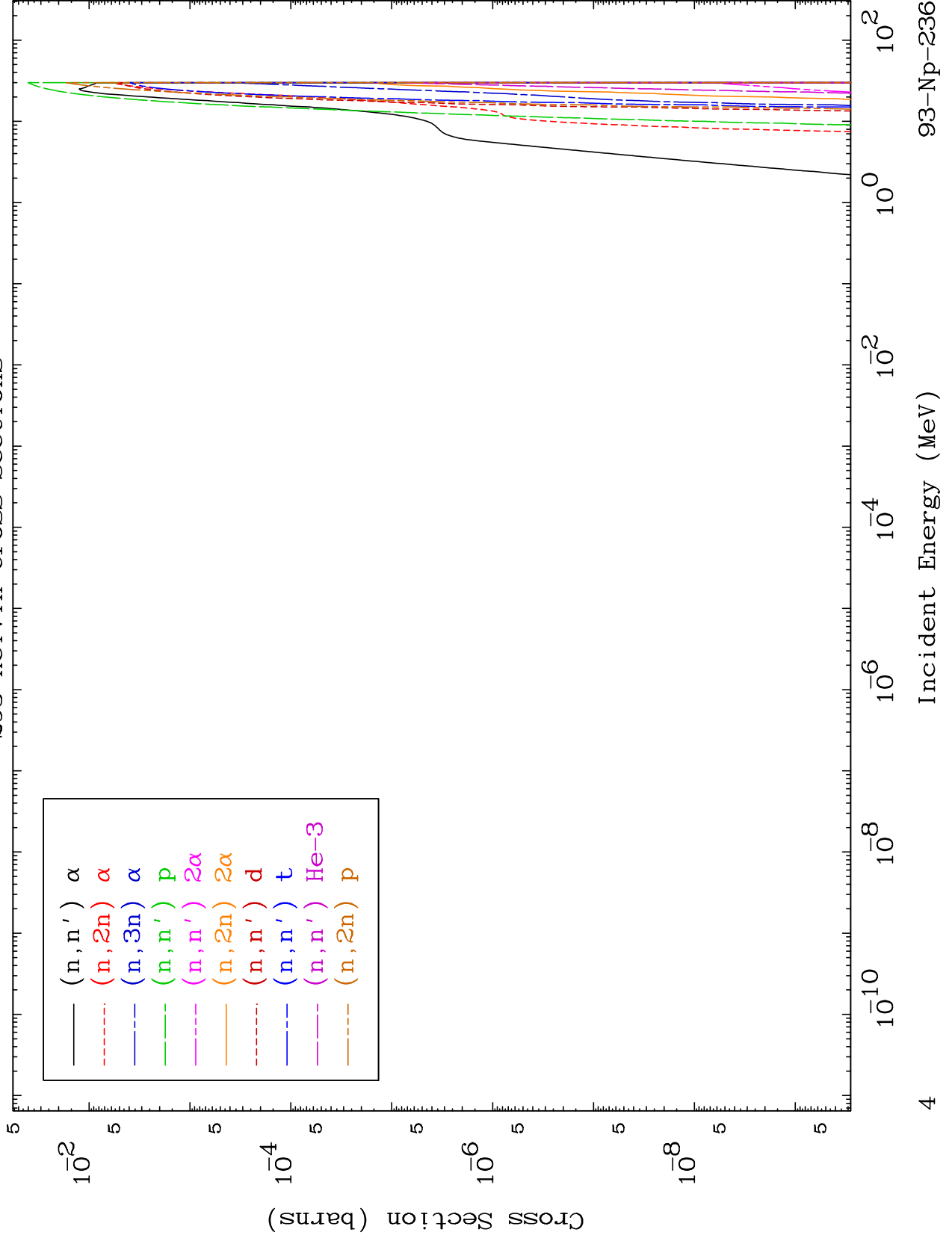
93-Np-236



MAT 9343

Charged Particle  
293 Kelvin Cross Sections

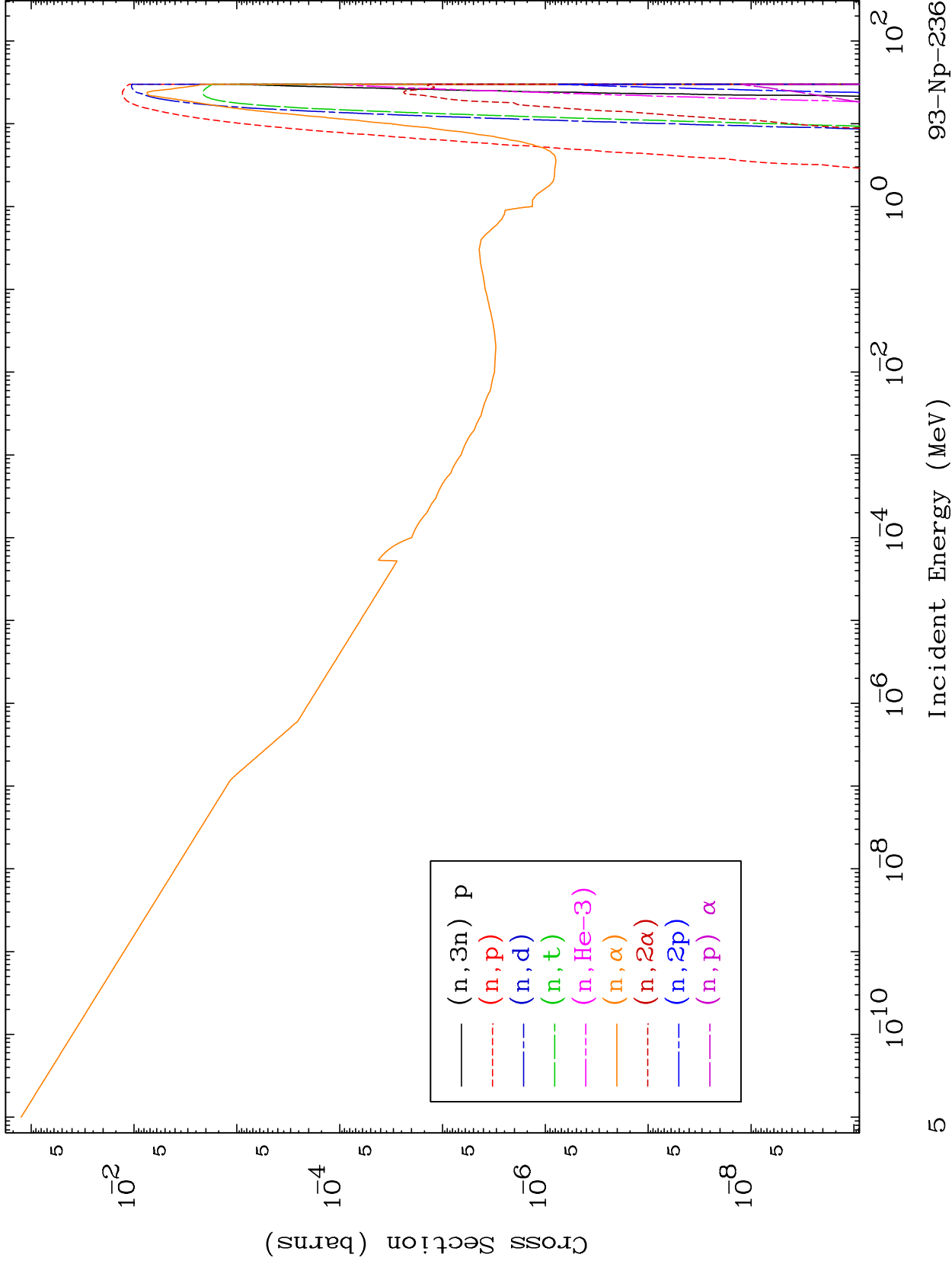
93-Np-236



MAT 9343

Charged Particle  
293 Kelvin Cross Sections

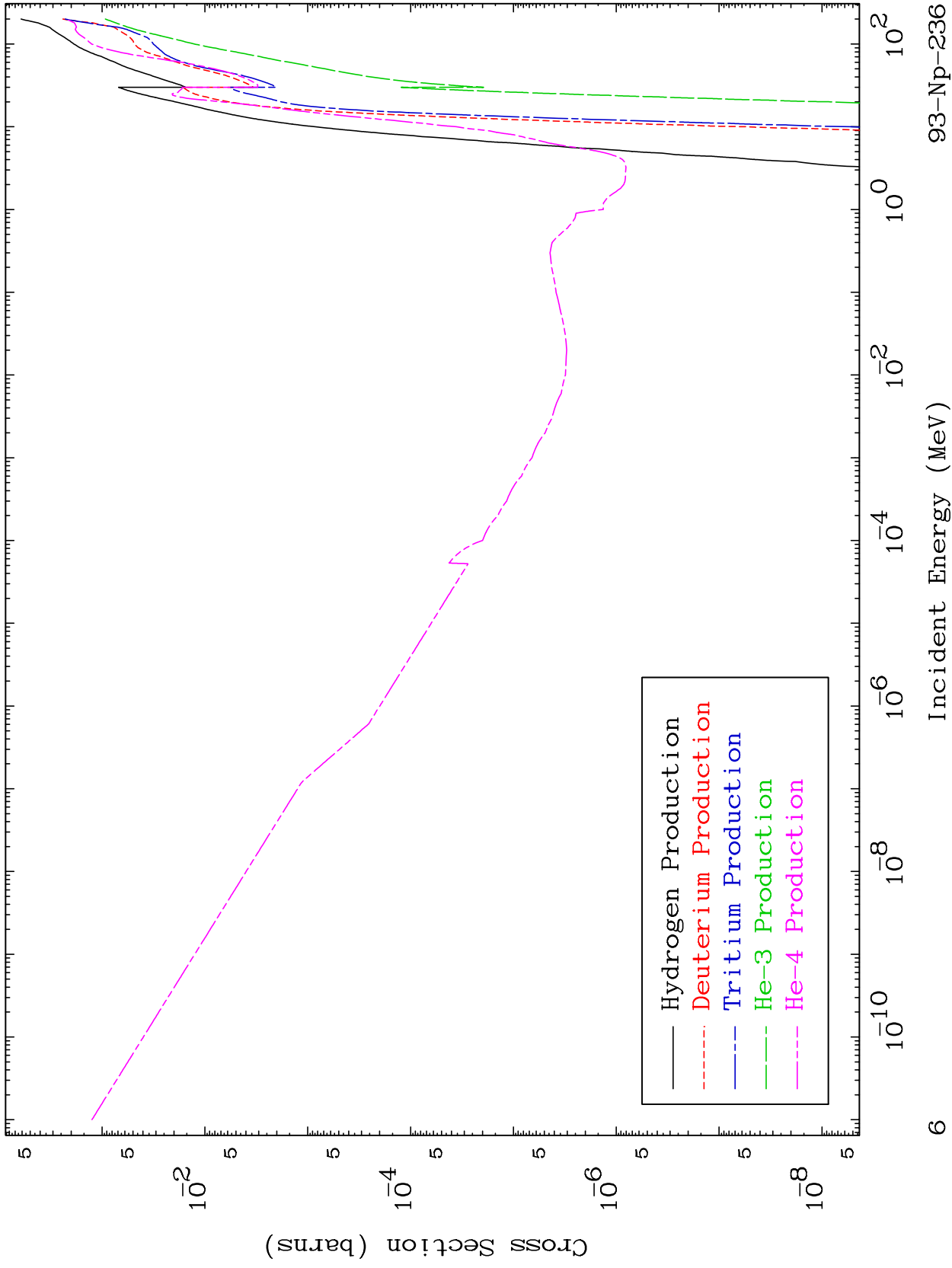
93-Np-236



MAT 9343

Particle Production  
293 Kelvin Cross Sections

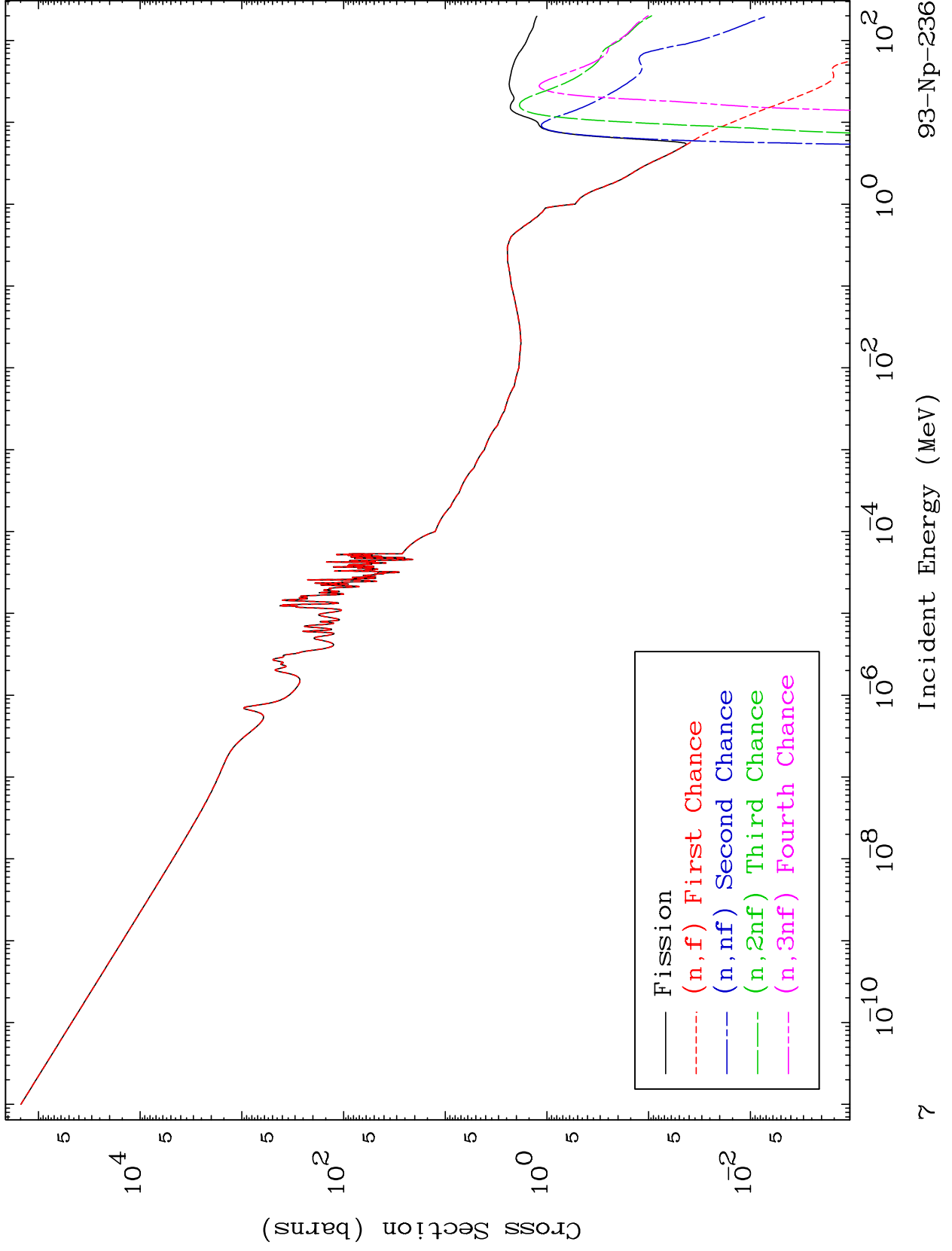
93-Np-236



MAT 9343

Fission  
293 Kelvin Cross Sections

93-Np-236



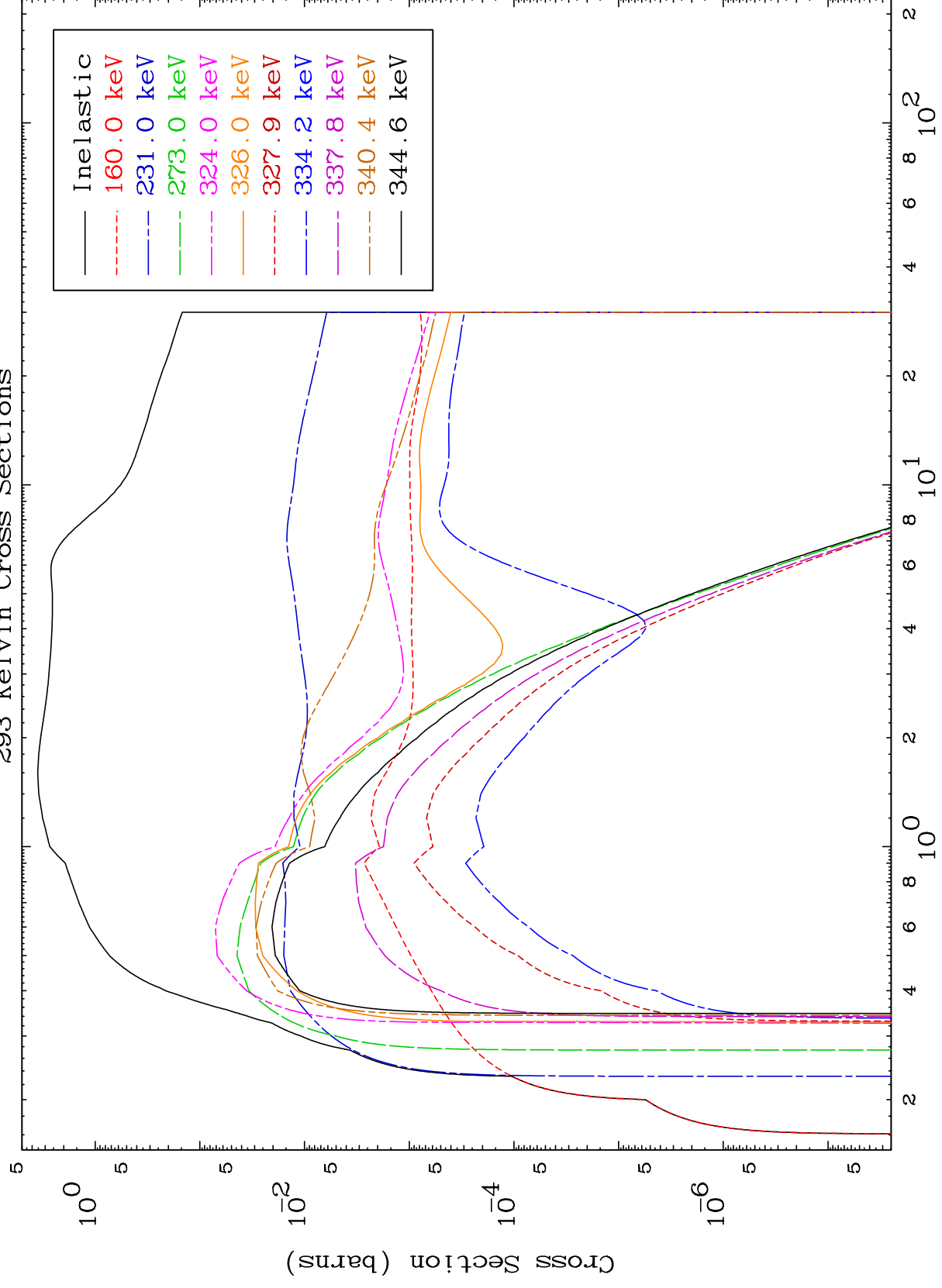


MAT 9343

(n,n') Level

93-Np-236

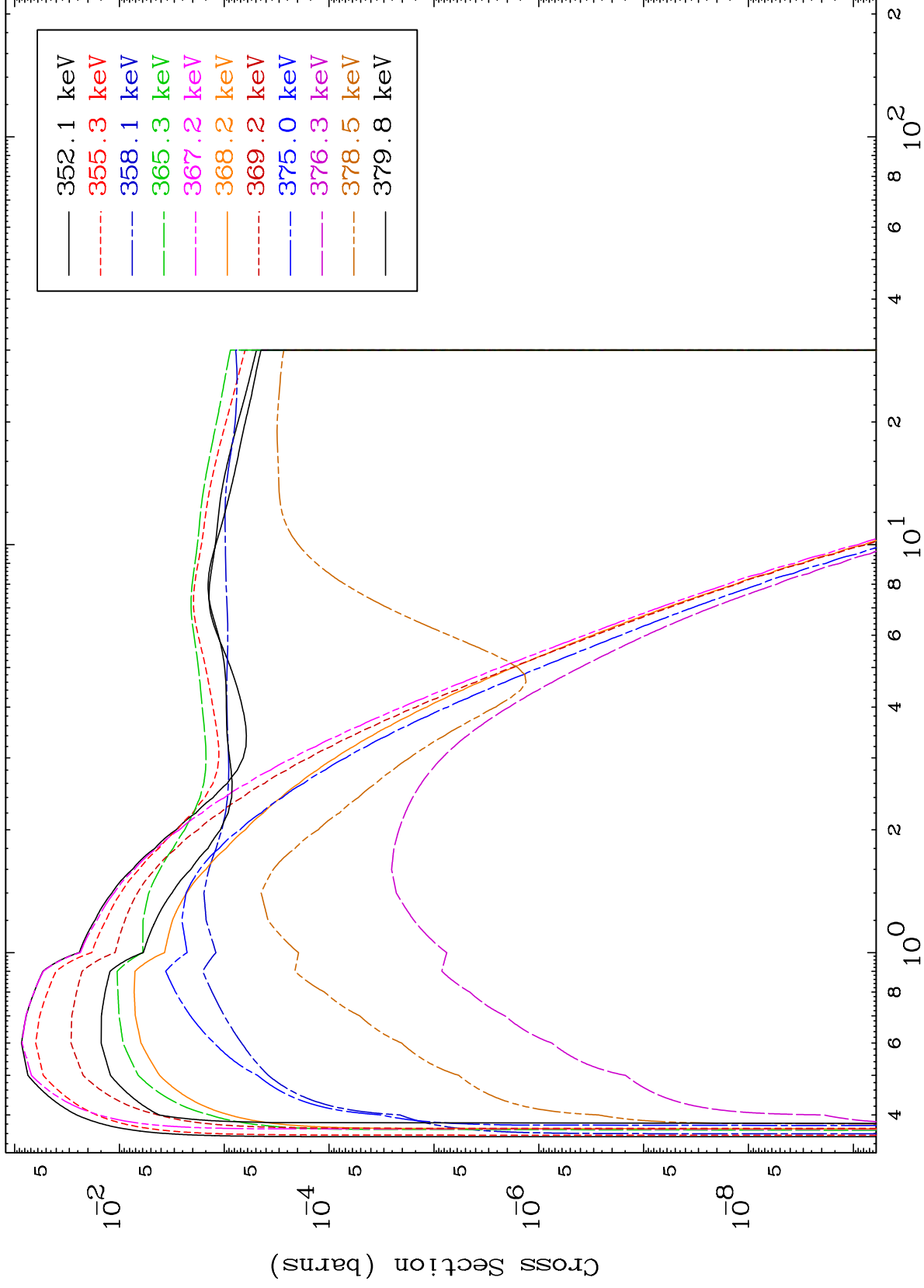
293 Kelvin Cross Sections



MAT 9343

(n,n') Level  
293 Kelvin Cross Sections

93-Np-236



9

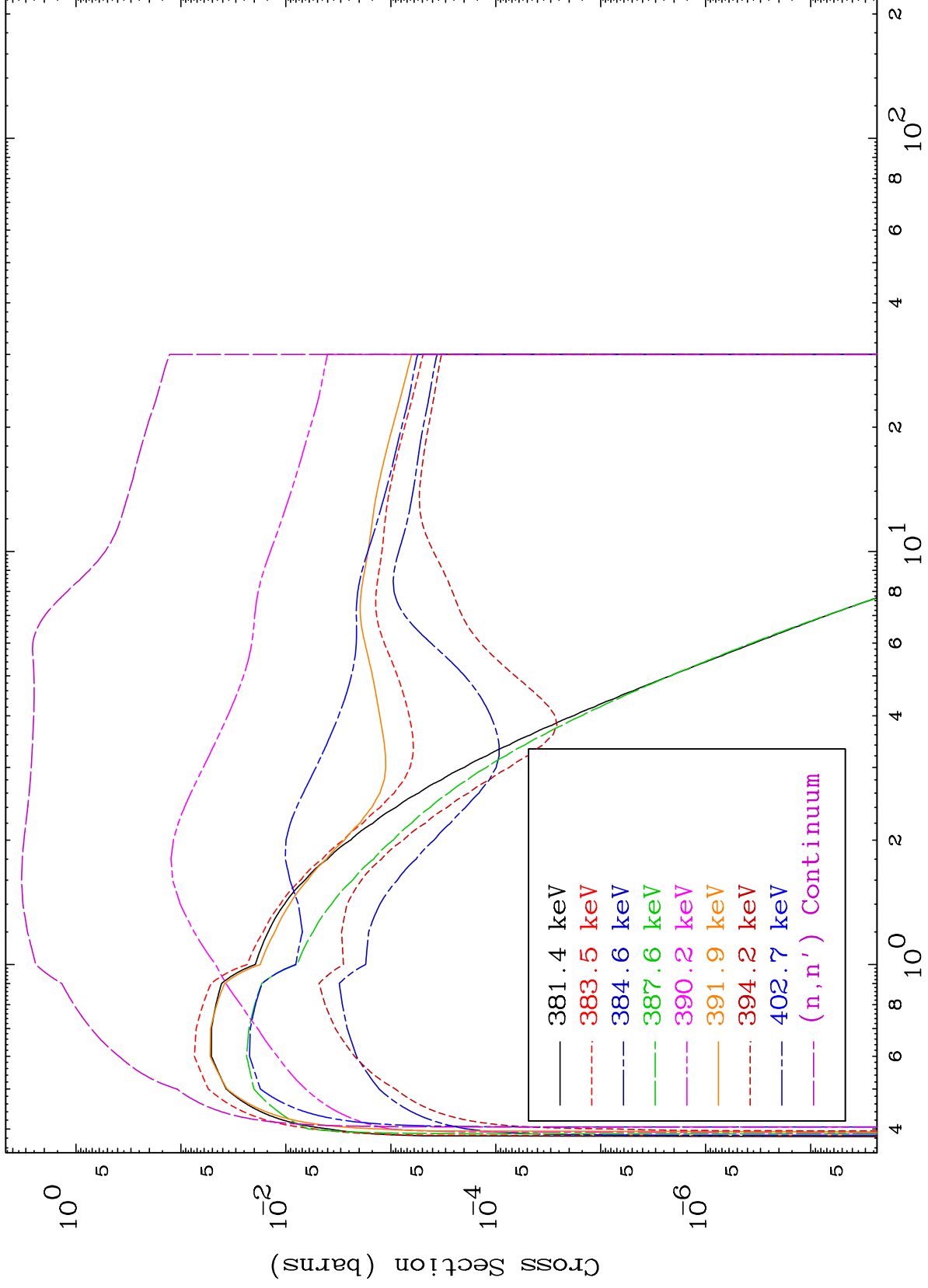
Incident Energy (MeV)

93-Np-236

MAT 9343

(n,n') Level  
293 Kelvin Cross Sections

93-Np-236



10

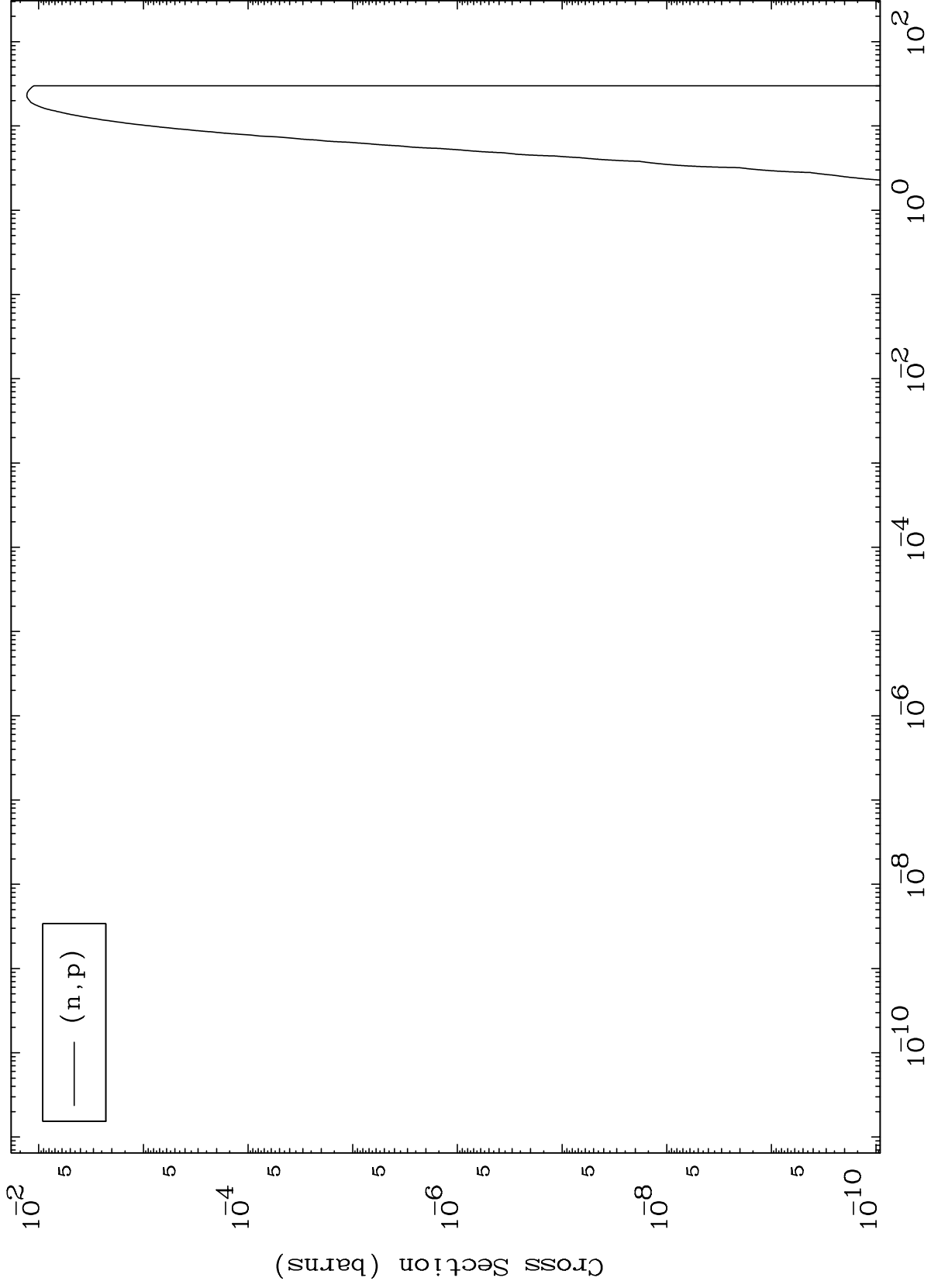
Incident Energy (MeV)

93-Np-236

MAT 9343

(n,p) Levels  
293 Kelvin Cross Sections

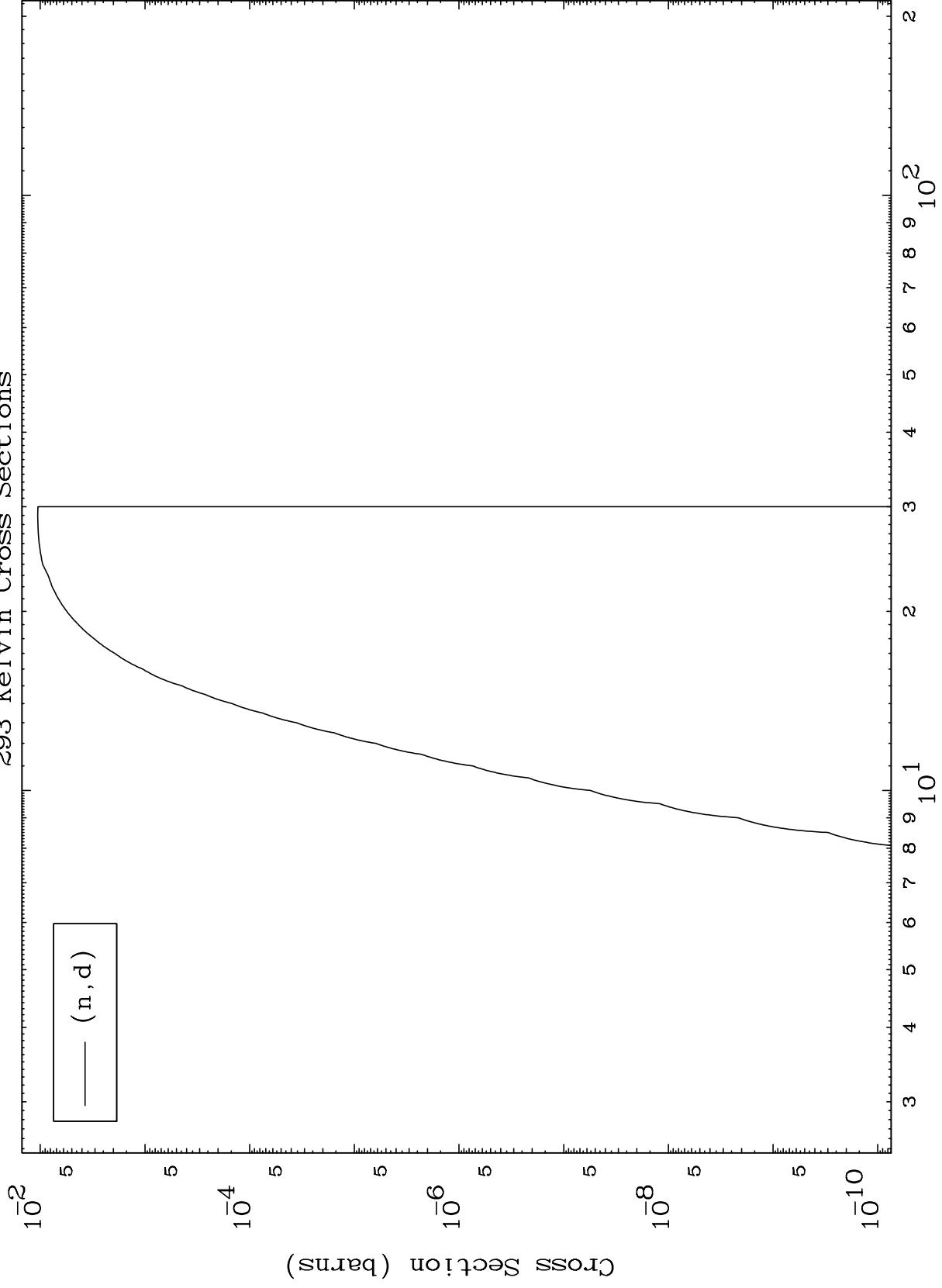
93-Np-236



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(n,d) Levels  
293 Kelvin Cross Sections

93-Np-236



12

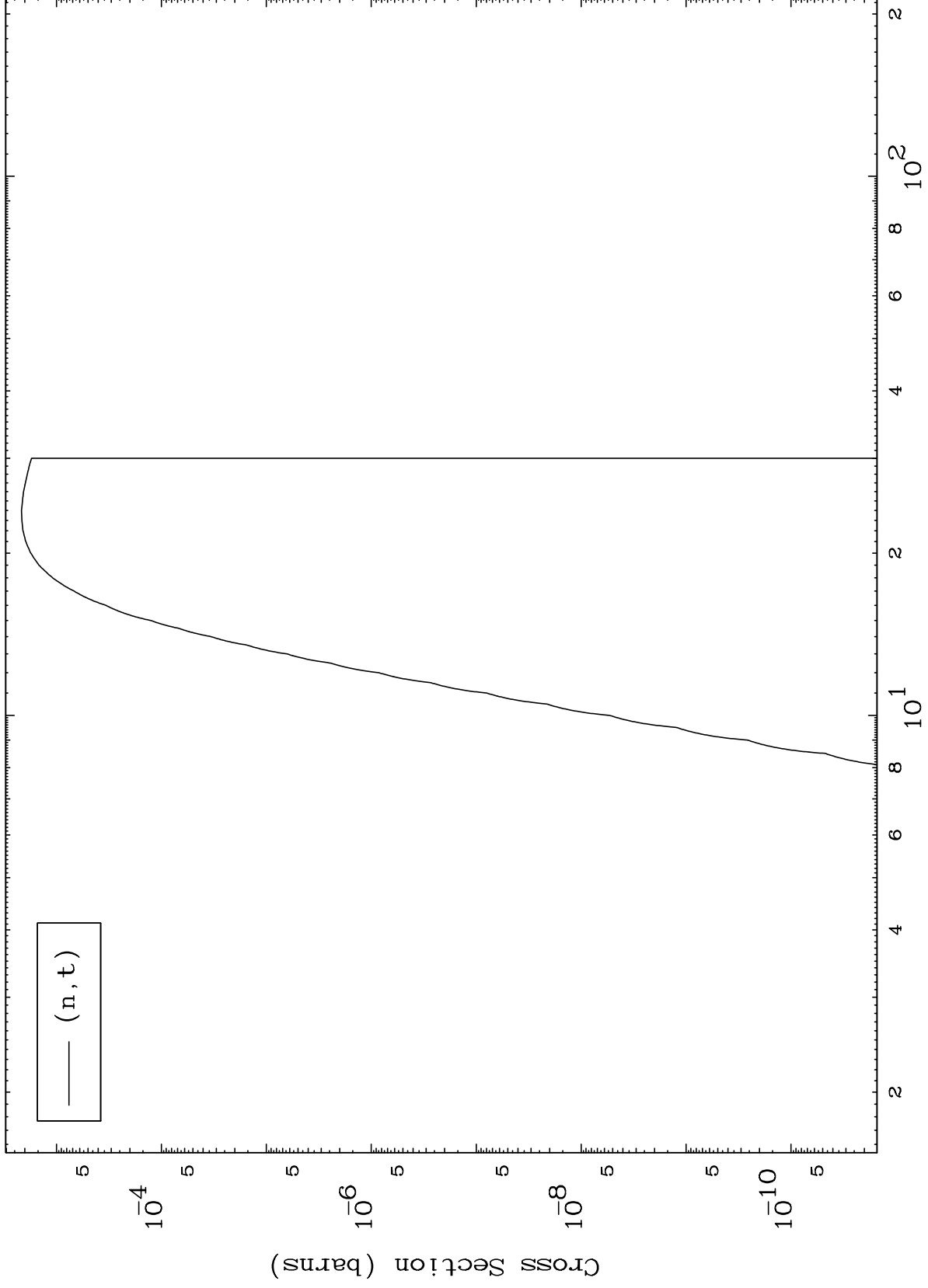
Incident Energy (MeV)

93-Np-236

MAT 9343

(n,t) Levels  
293 Kelvin Cross Sections

93-Np-236



13

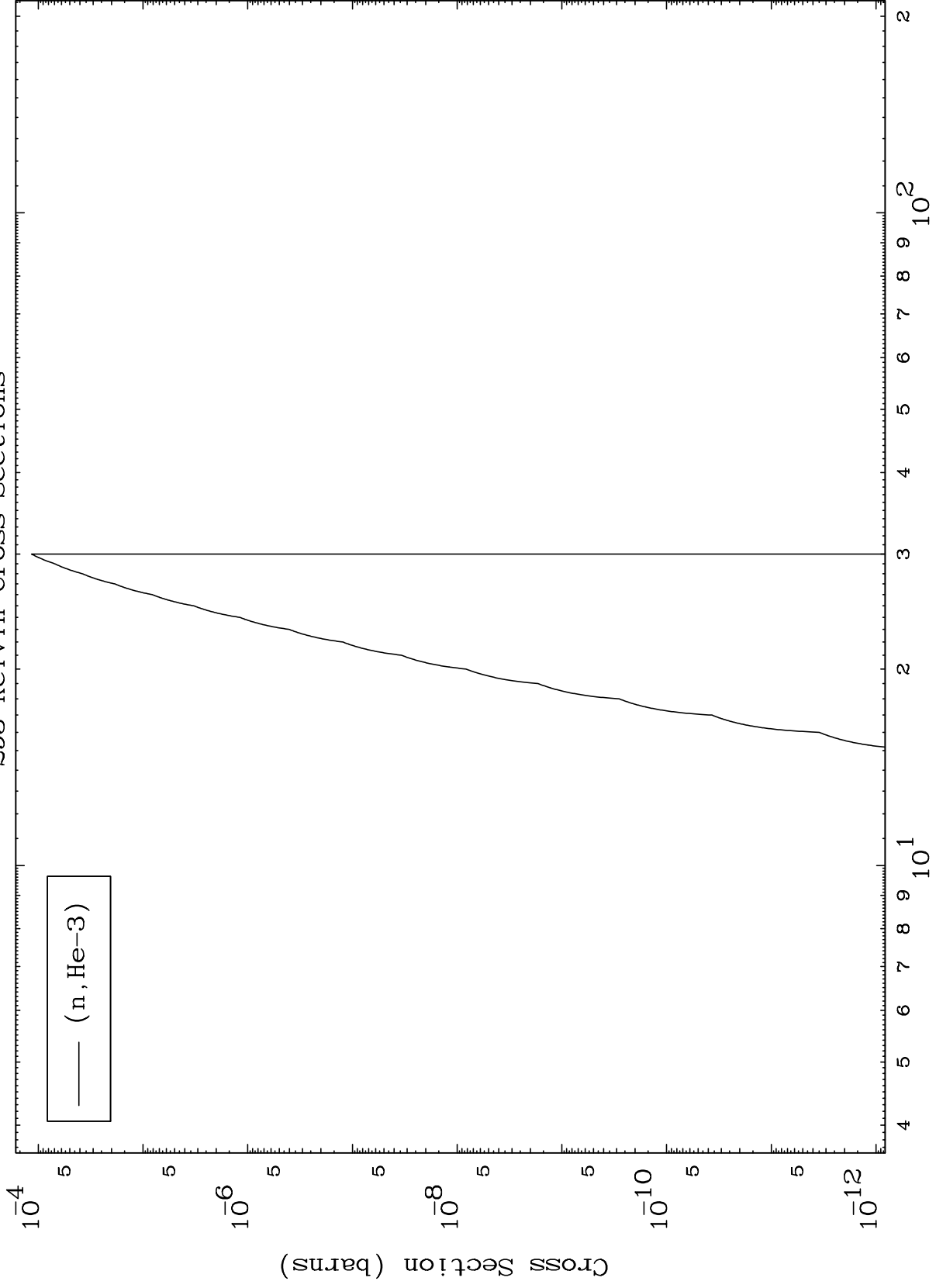
Incident Energy (MeV)

93-Np-236

MAT 9343

(n,He3) Levels  
293 Kelvin Cross Sections

93-Np-236



(n, He-3)

14

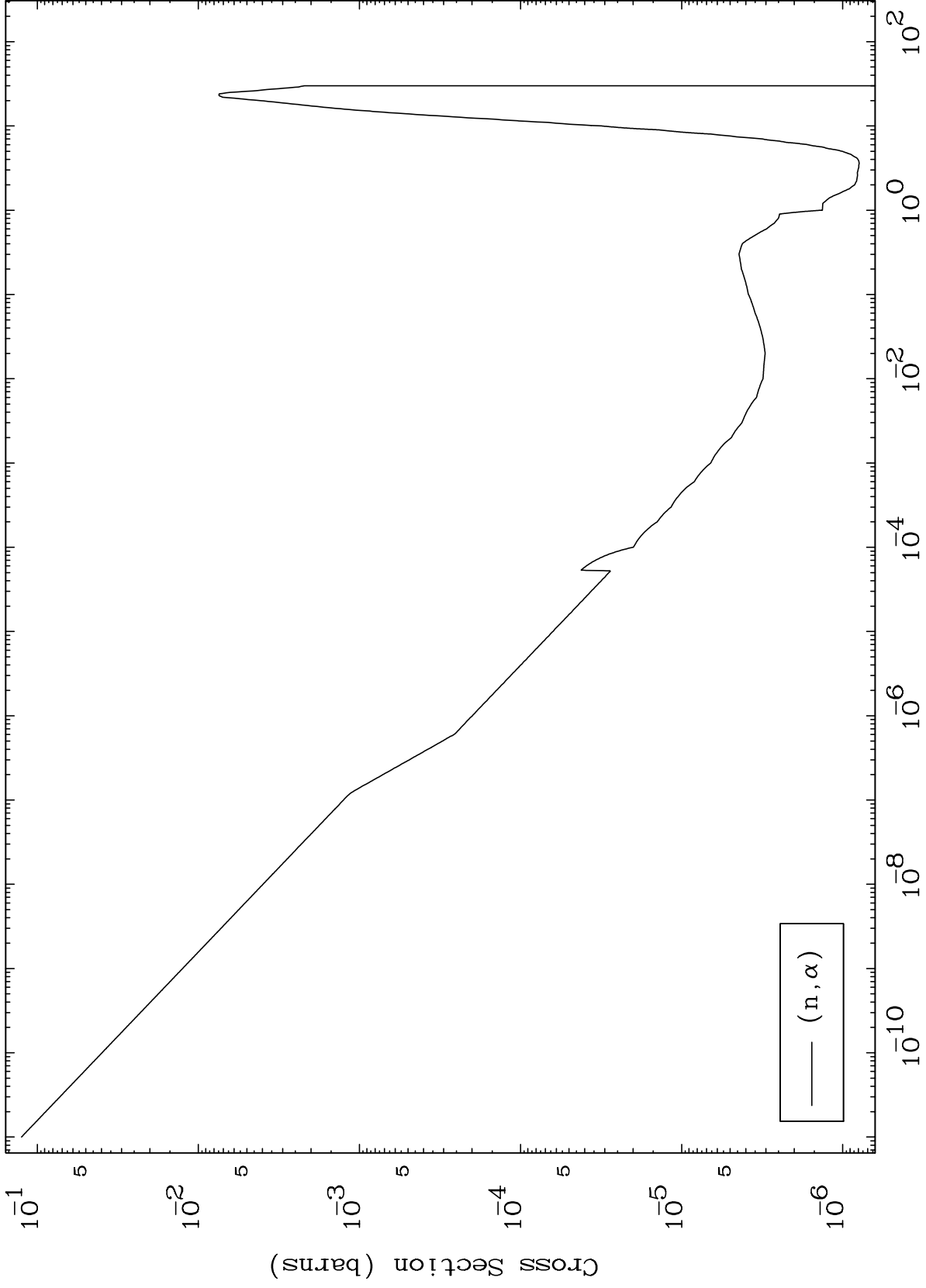
Incident Energy (MeV)

93-Np-236

MAT 9343

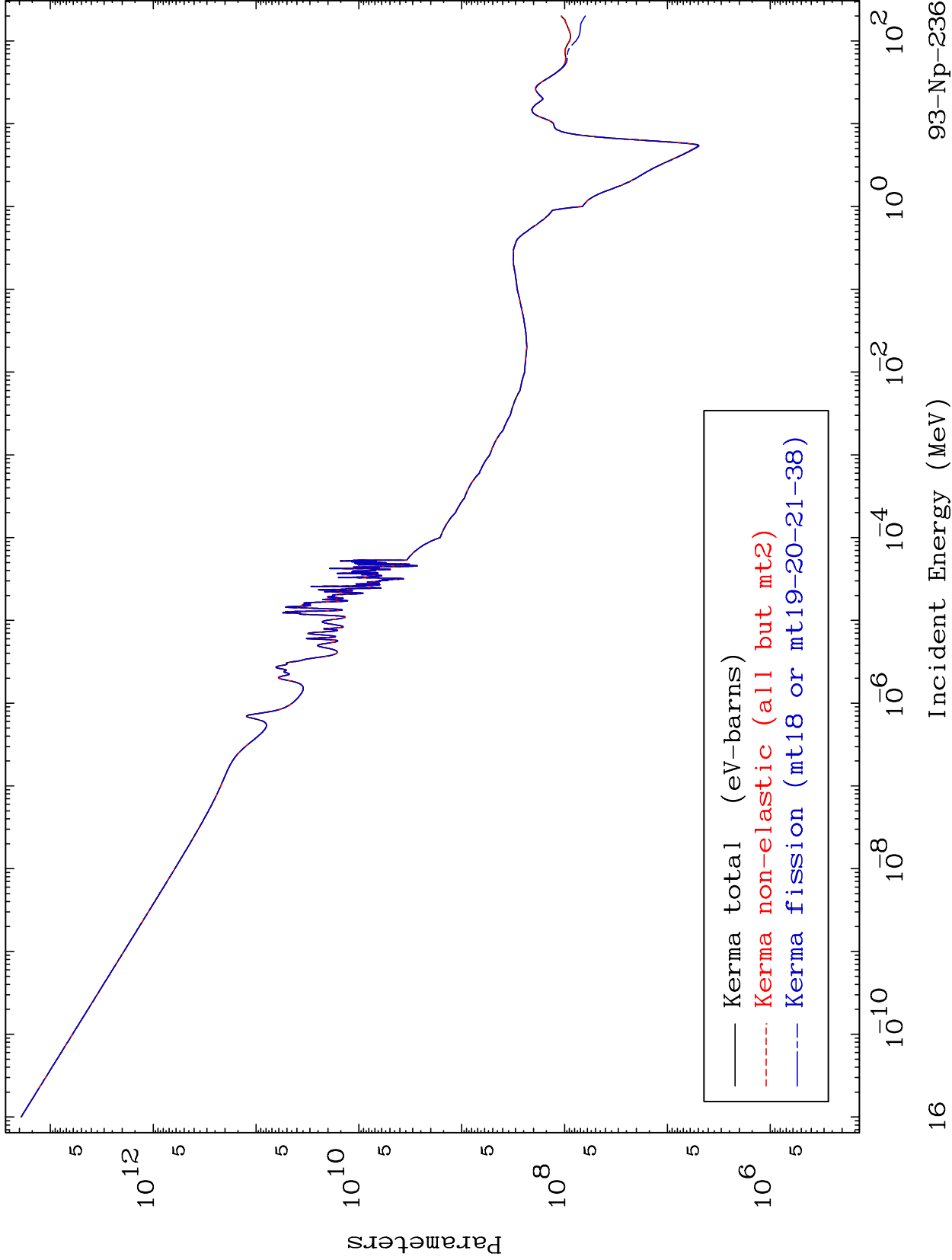
(n,  $\alpha$ ) Levels  
293 Kelvin Cross Sections

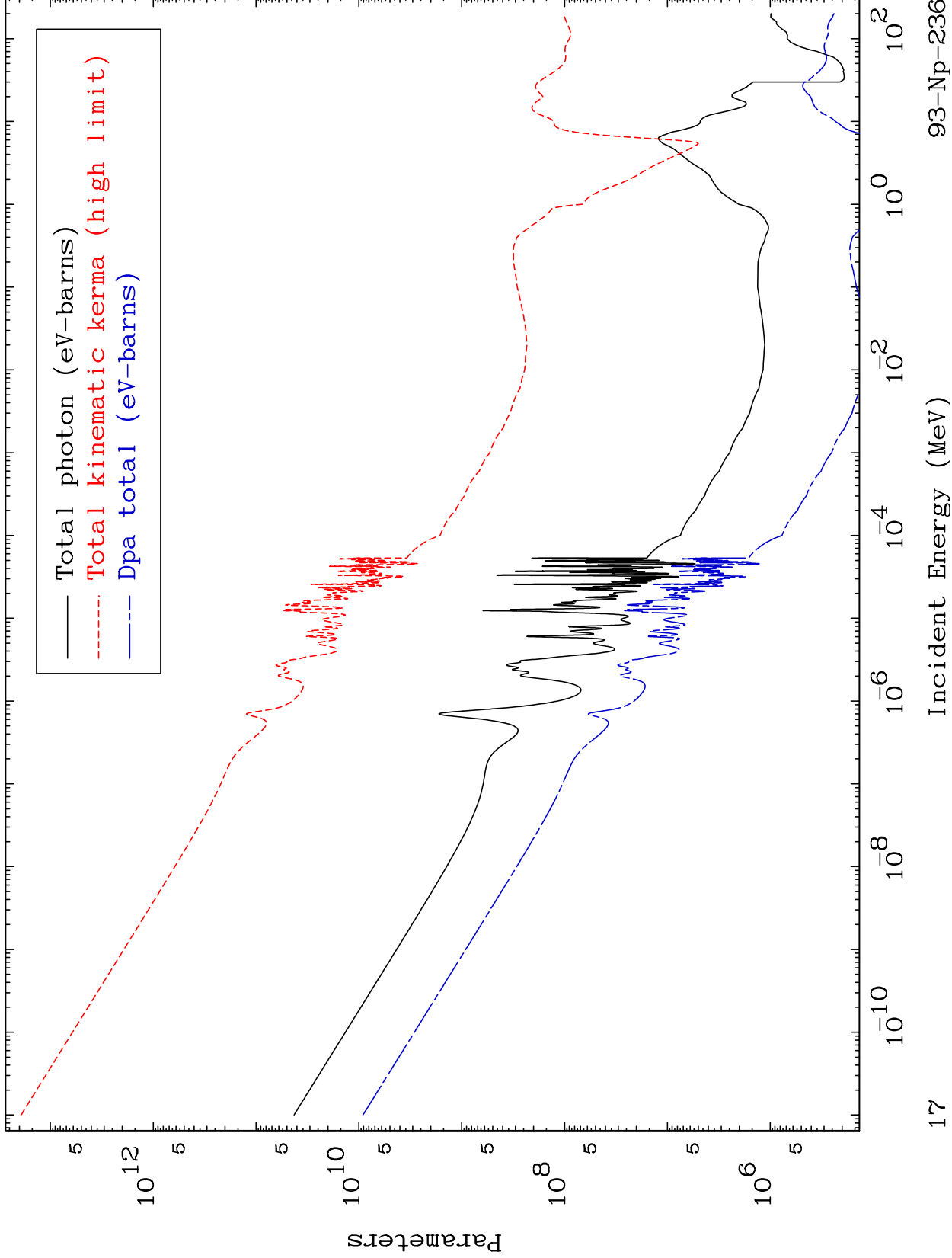
93-Np-236



(n,  $\alpha$ )



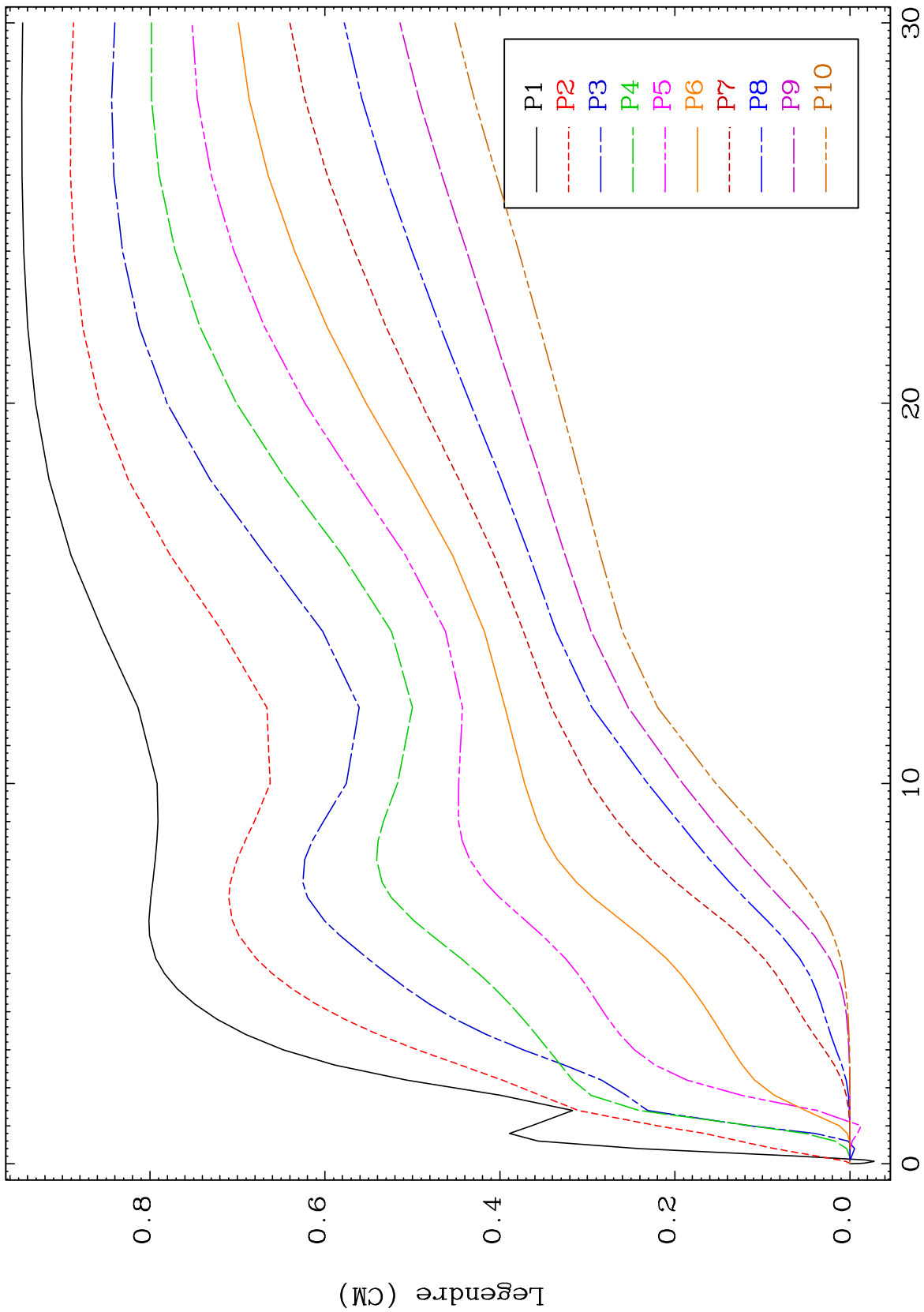




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93-Np-236

Elastic Legendre Coefficients



93-Np-236

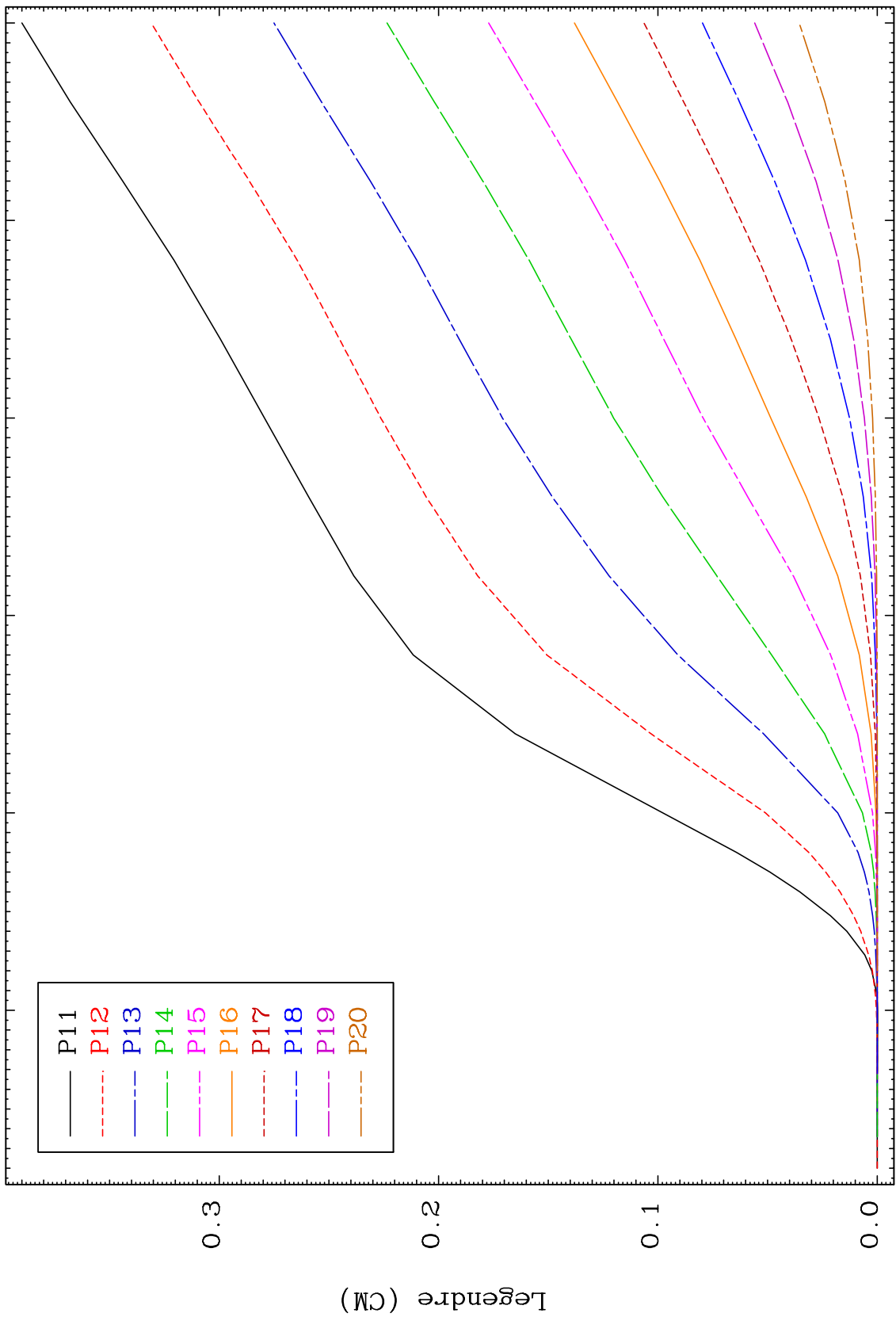
Incident Energy (MeV)

18

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Elastic Legendre Coefficients

<sup>93</sup>Np-236



19

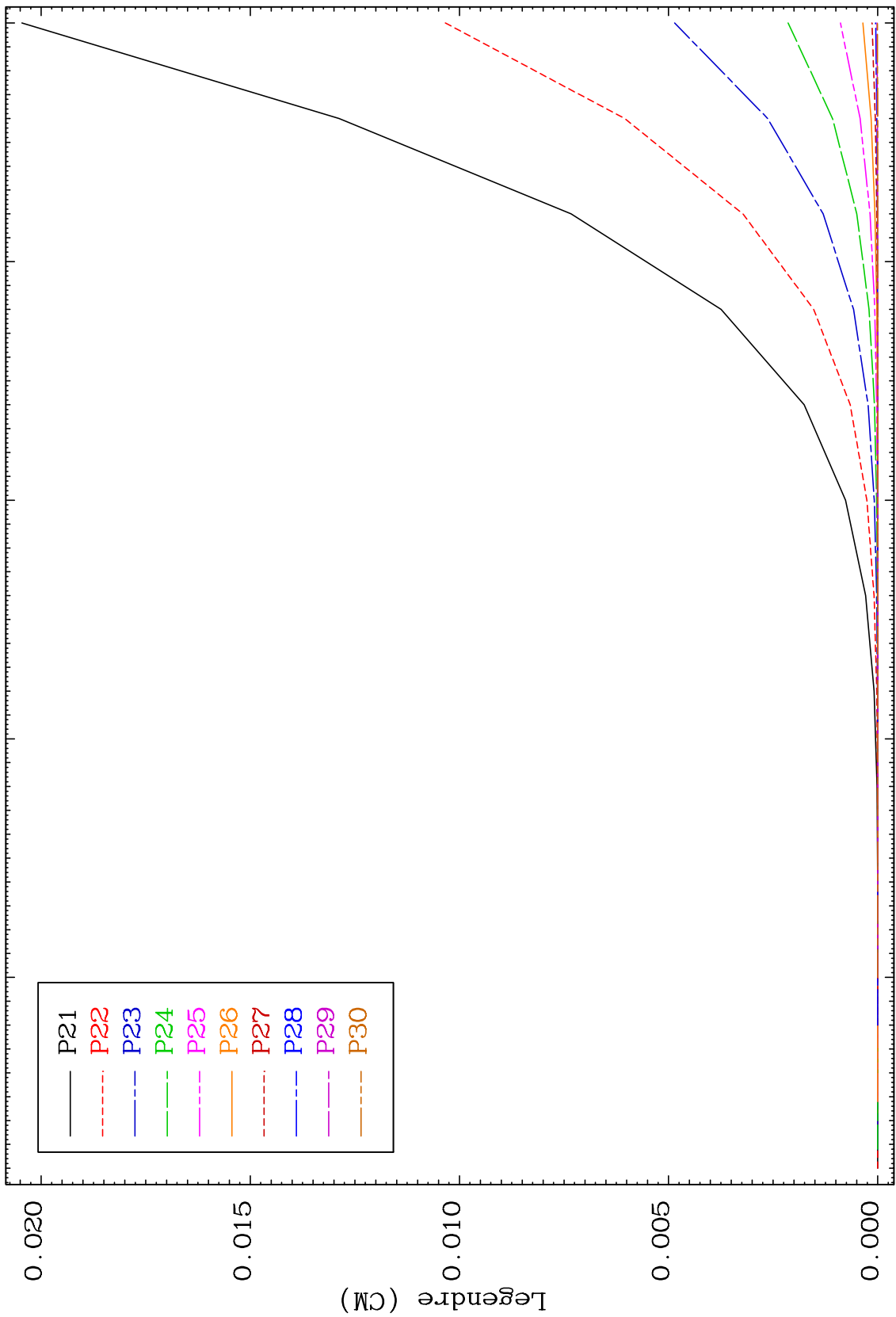
Incident Energy (MeV)

<sup>93</sup>Np-236

MAT 9343

Elastic  
Legendre Coefficients

93-Np-236



20

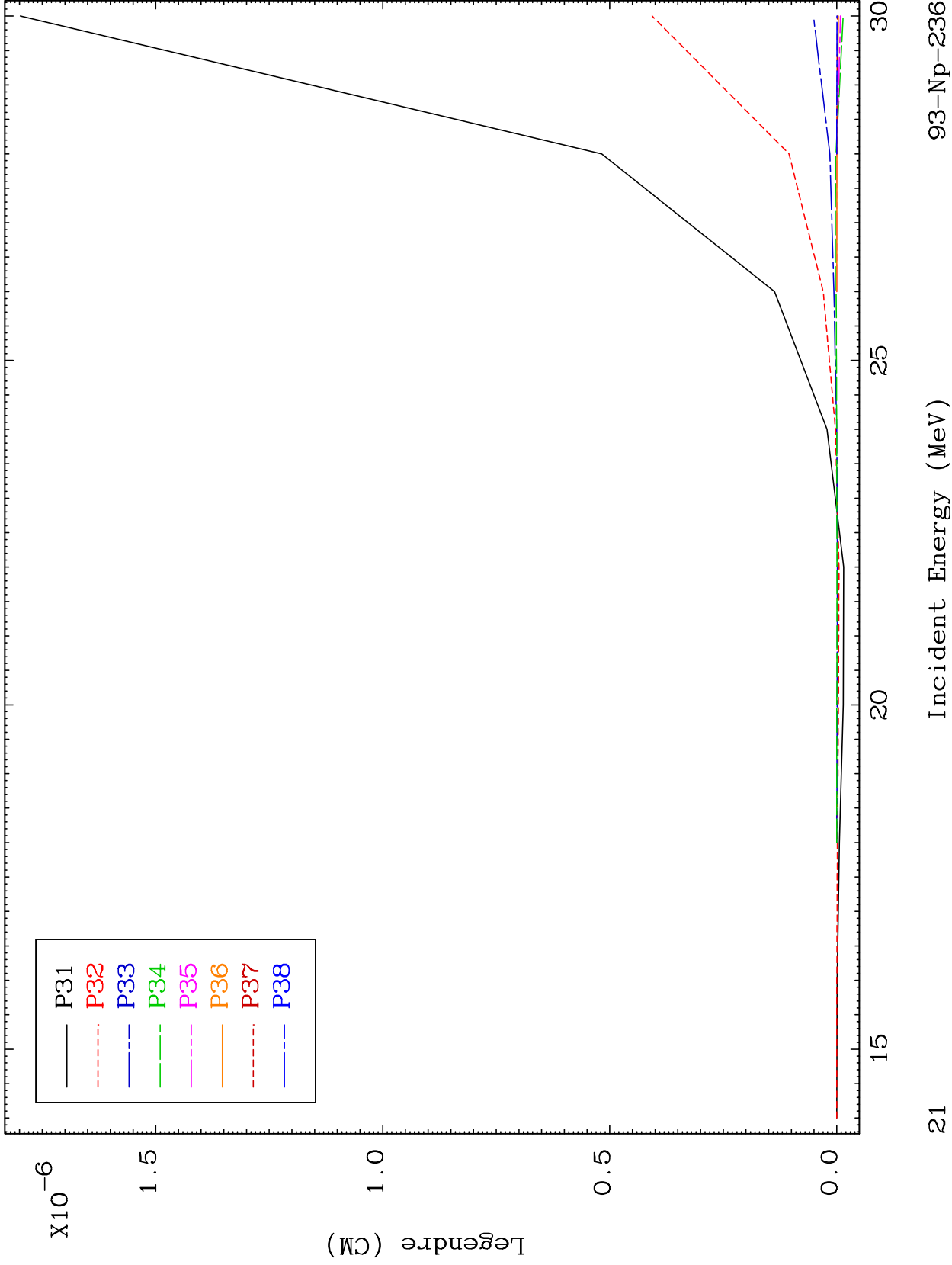
Incident Energy (MeV)

93-Np-236

MAT 9343

Elastic  
Legendre Coefficients

93-Np-236

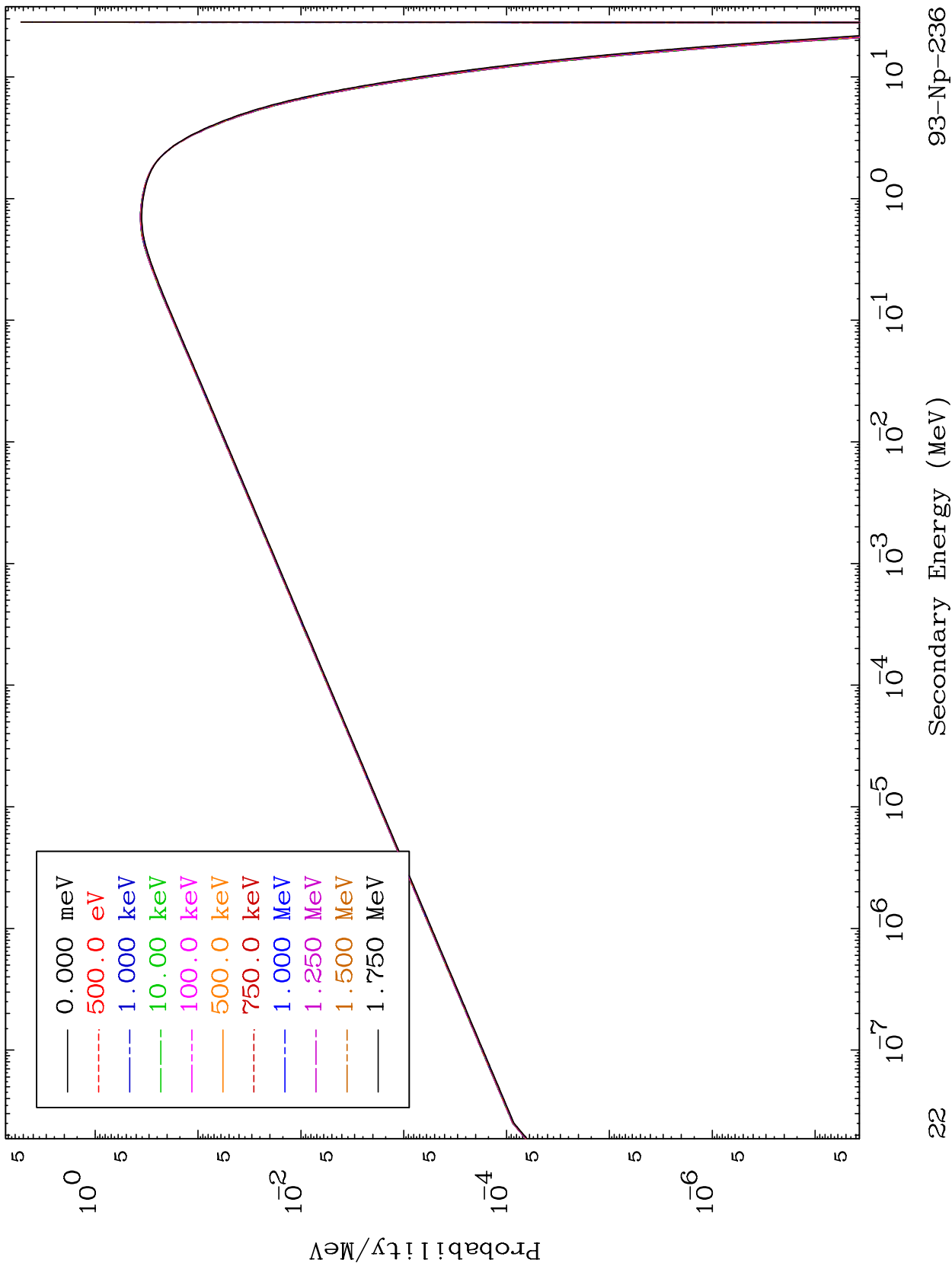


21

Incident Energy (MeV)

93-Np-236

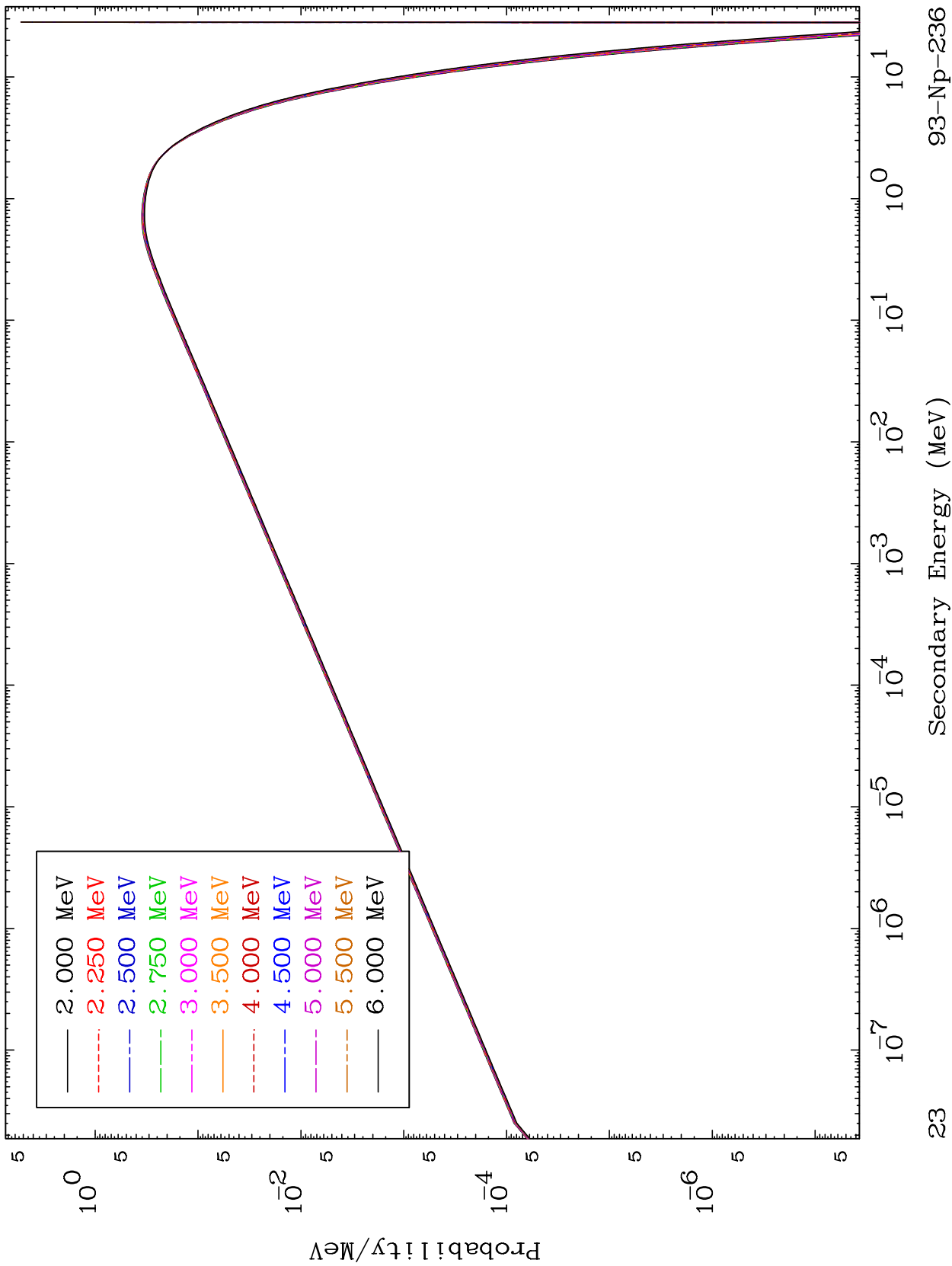
Fission Energy Distribution



MAT 9343

### Fission Energy Distribution

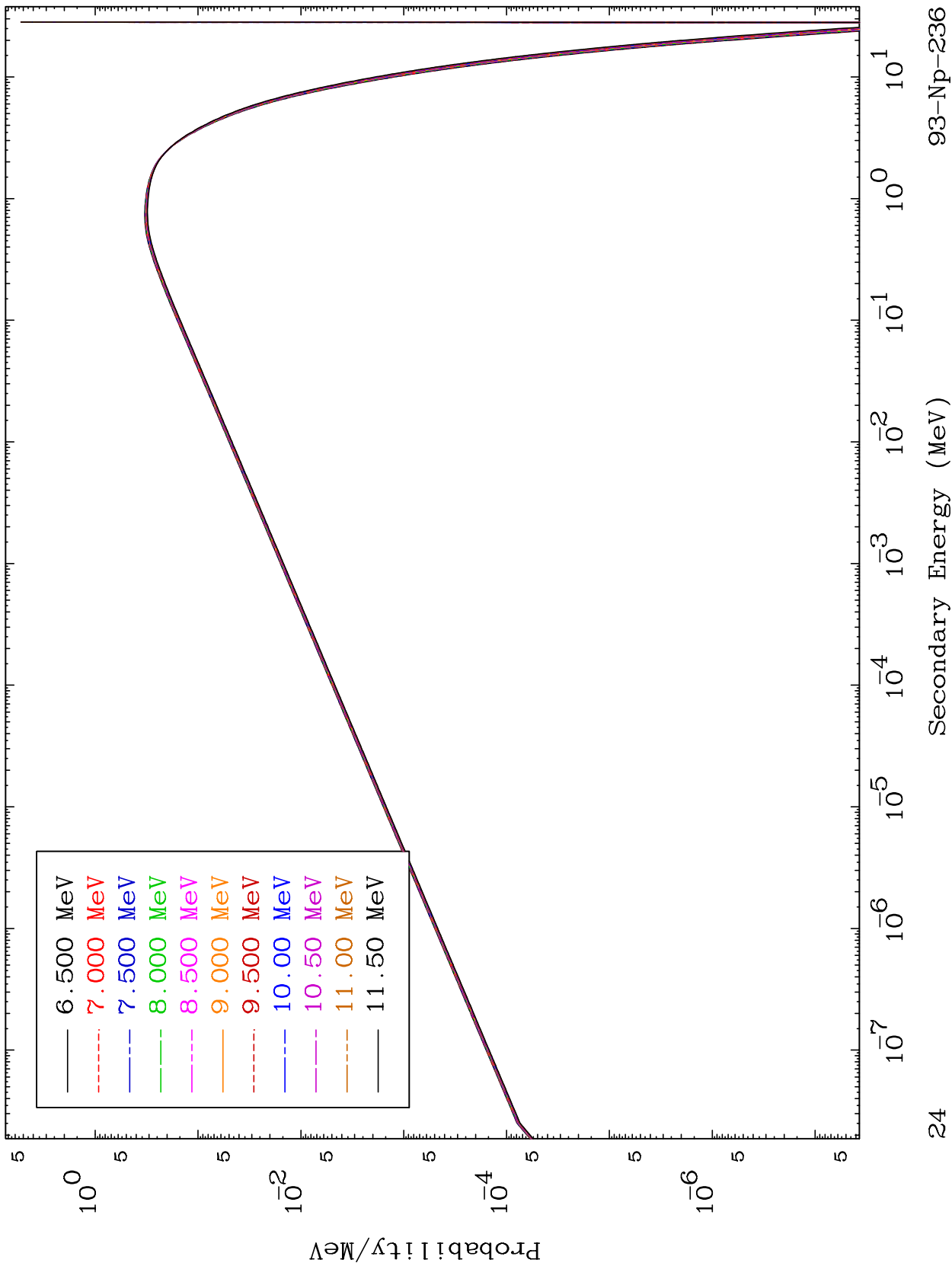
93-Np-236



93-Np-236



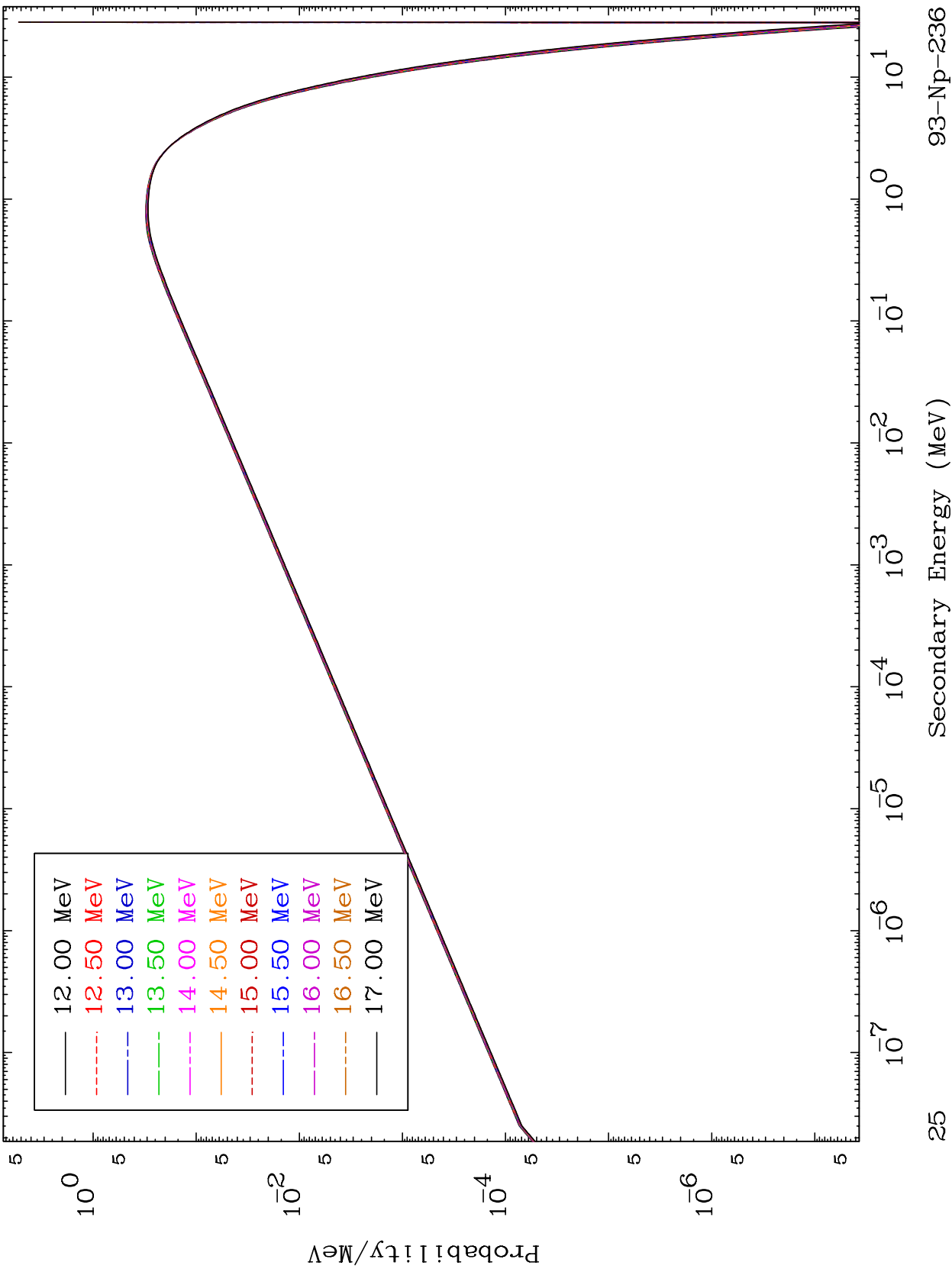
### Fission Energy Distribution



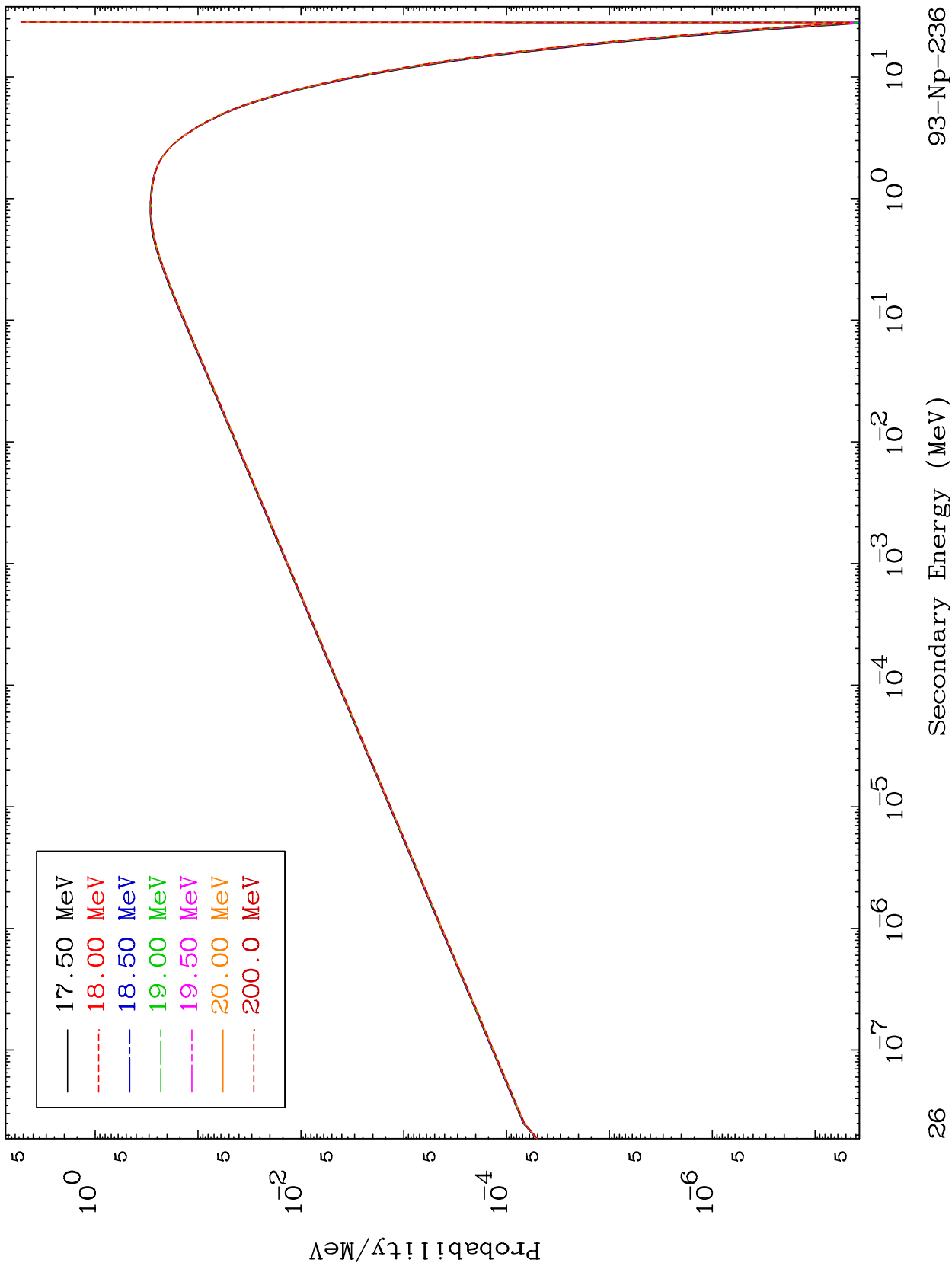
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### Fission Energy Distribution

93-Np-236



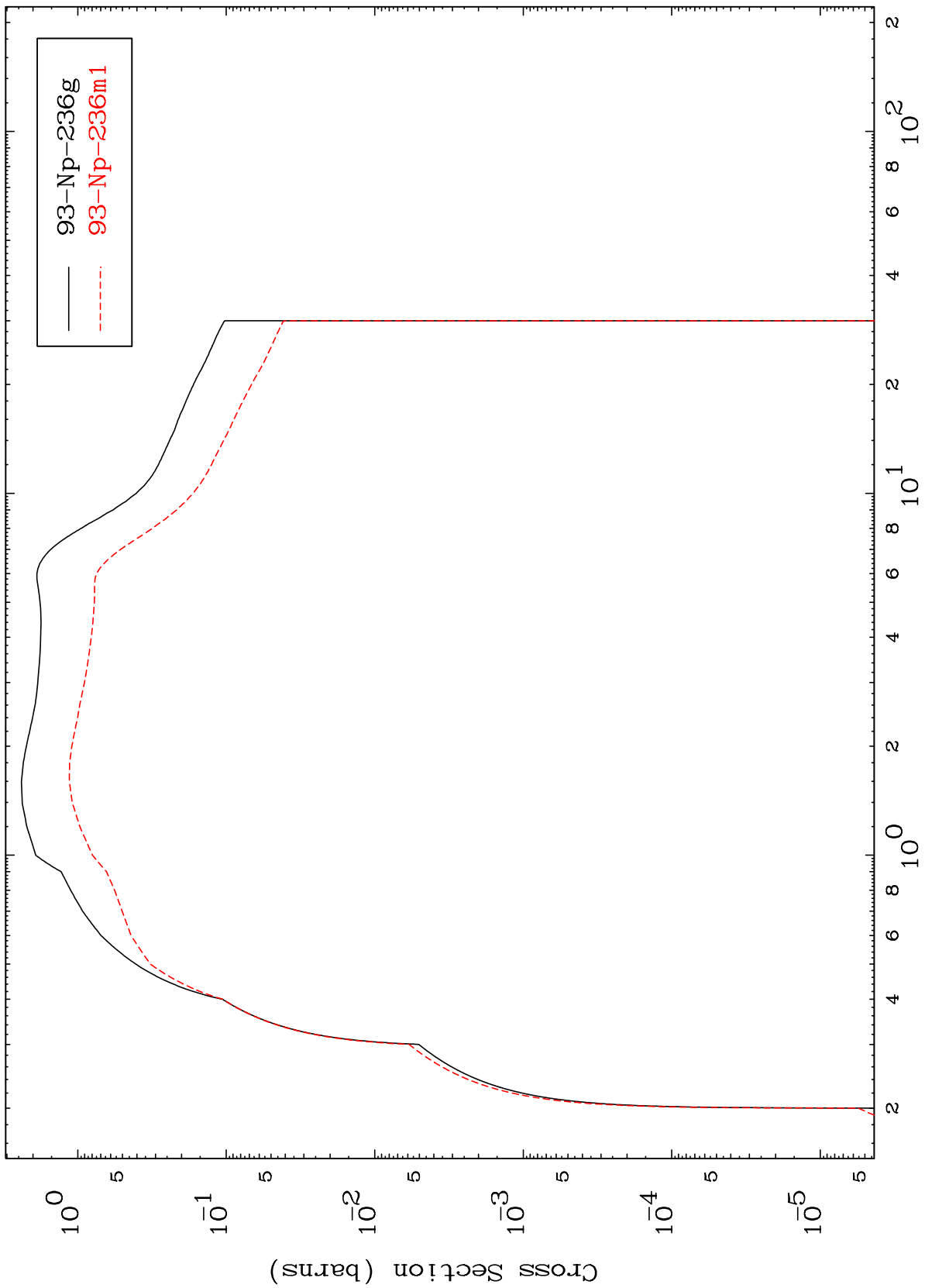
Fission  
Energy Distribution



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93-Np-236

Inelastic  
Radionuclide Production Cross Section



27

Incident Energy (MeV)

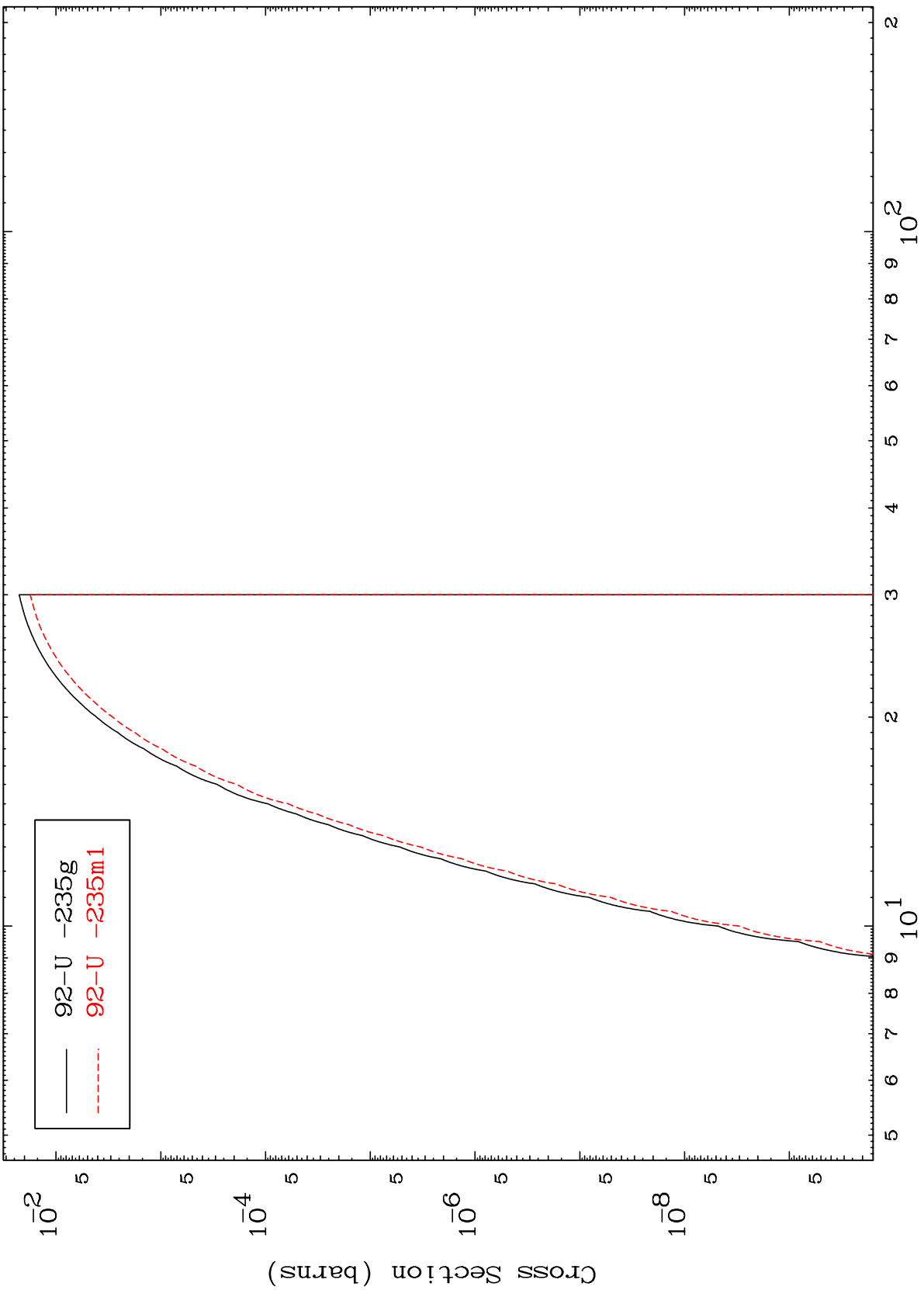
93-Np-236

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(n,n') p

93-Np-236

Radionuclide Production Cross Section



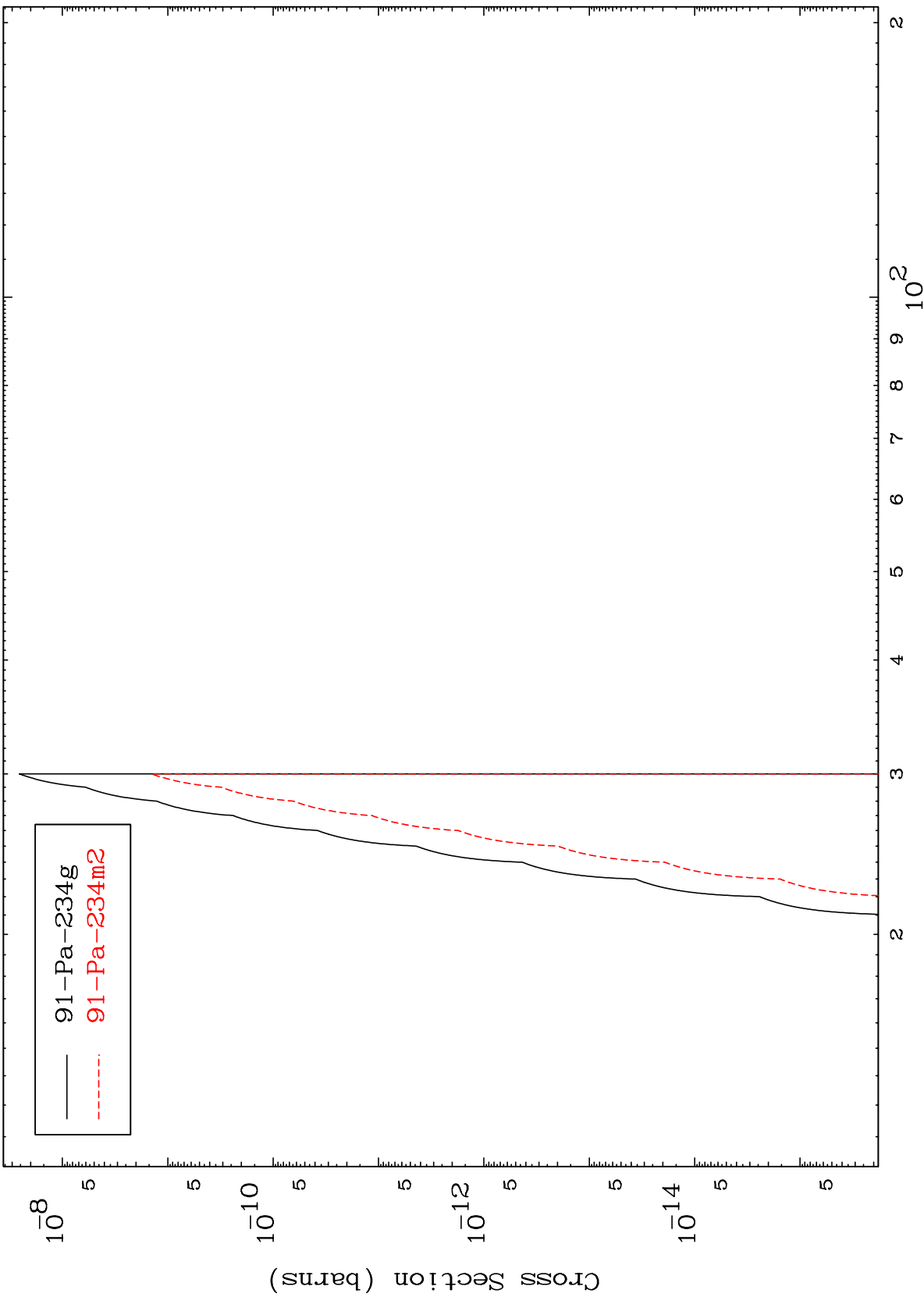
Legend:  
— 92-U -235g  
- - - 92-U -235m1

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(n,2n) p

93-Np-236

Radionuclide Production Cross Section

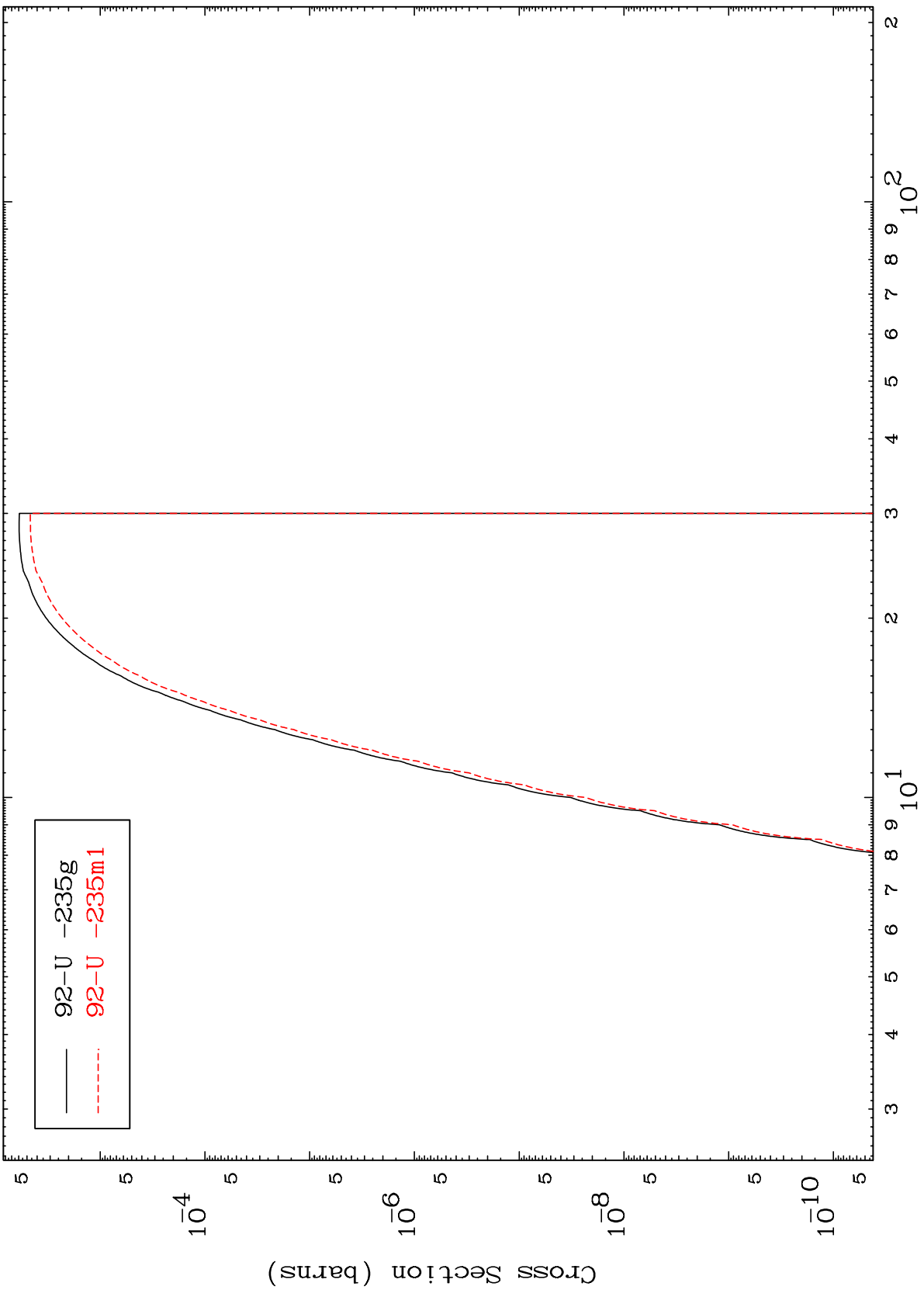


91-Pa-234g  
91-Pa-234m2

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93-Np-236

(n,d)  
Radionuclide Production Cross Section



30

Incident Energy (MeV)

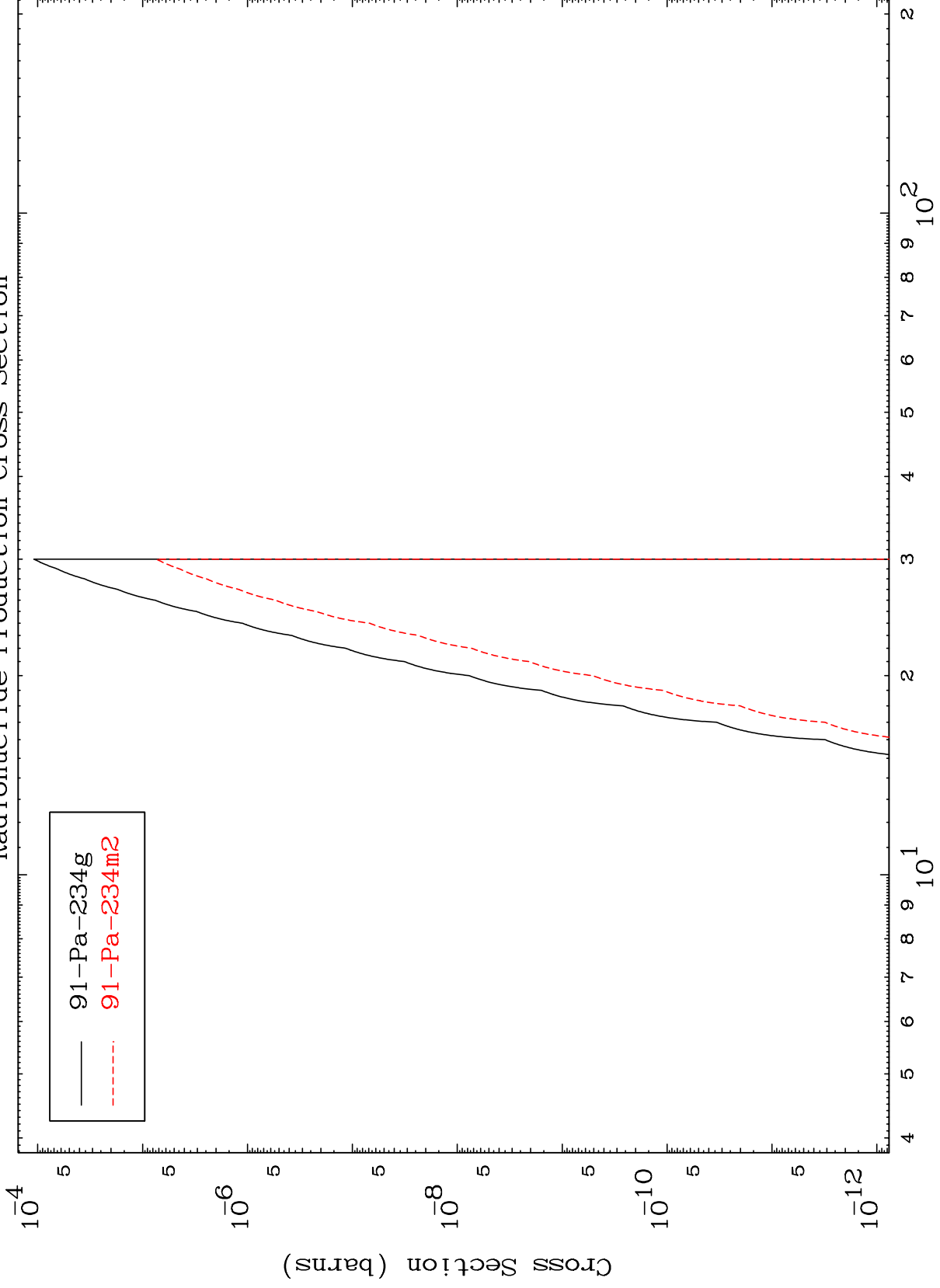
93-Np-236

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(n,He-3)

93-Np-236

Radionuclide Production Cross Section



31

Incident Energy (MeV)

93-Np-236