

Program EVALPLOT
(Version 2015-2)

by

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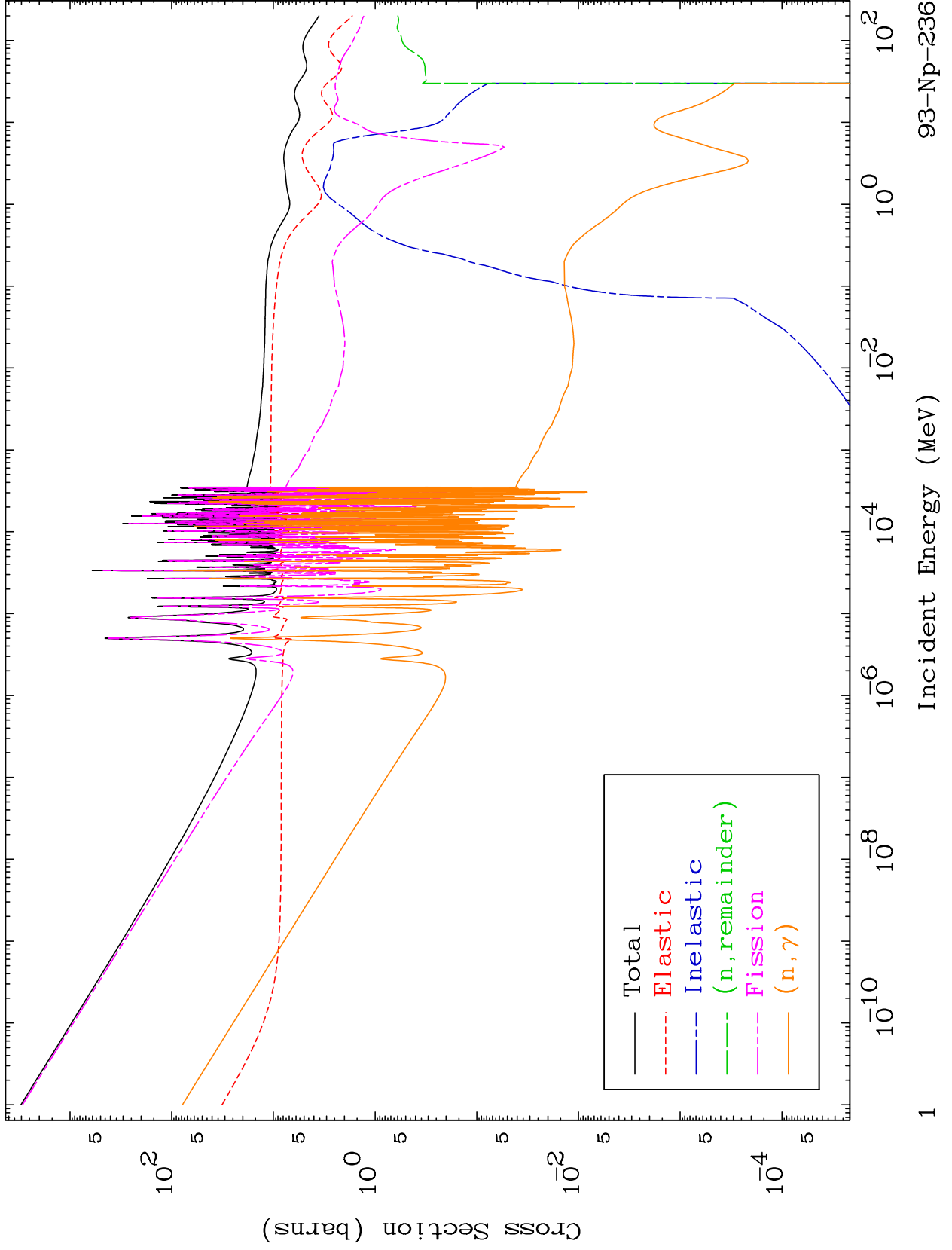
Press Mouse Button to Start

MAT 9344

Major

293 Kelvin Cross Sections

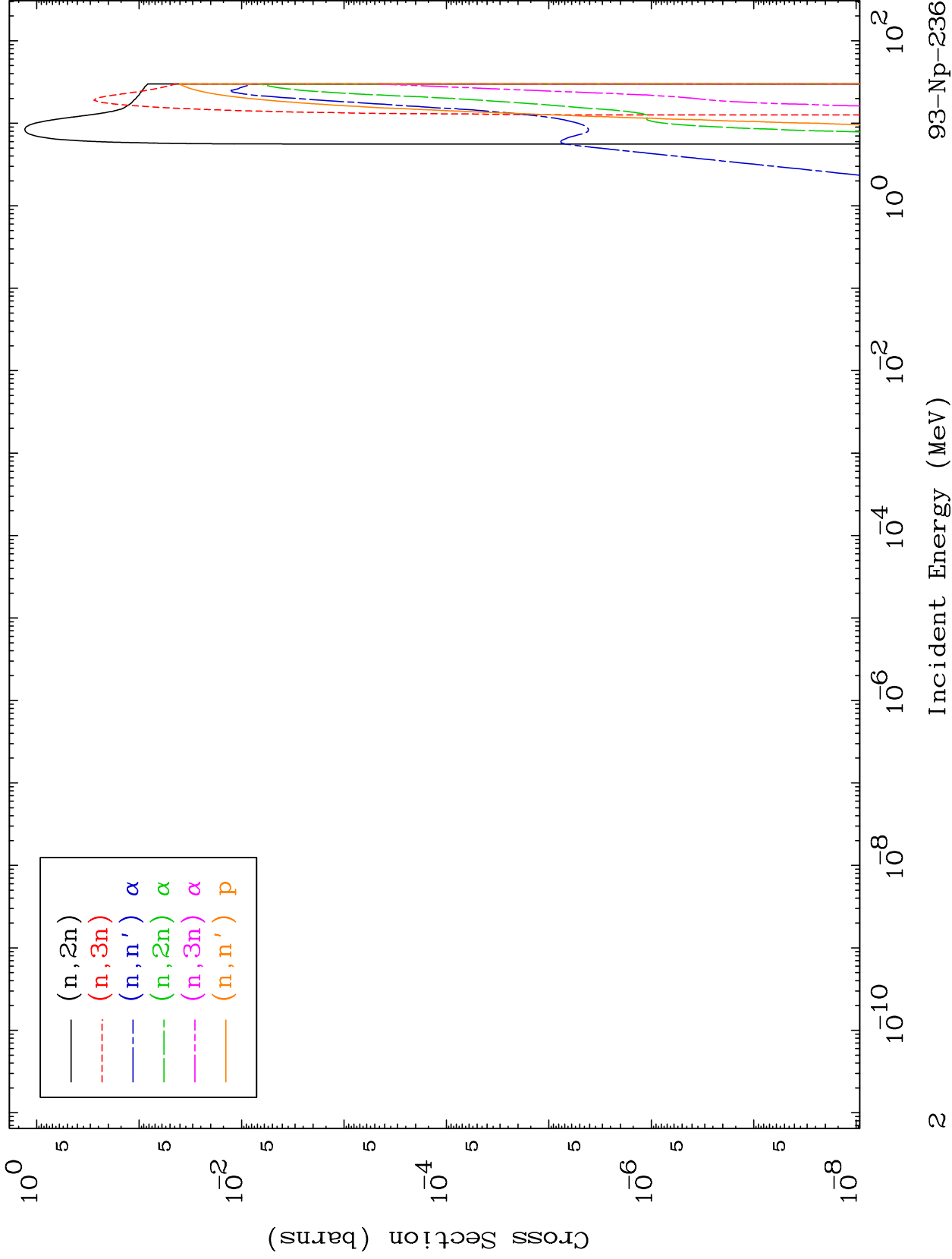
93-Np-236



MAT 9344

Neutron Production
293 Kelvin Cross Sections

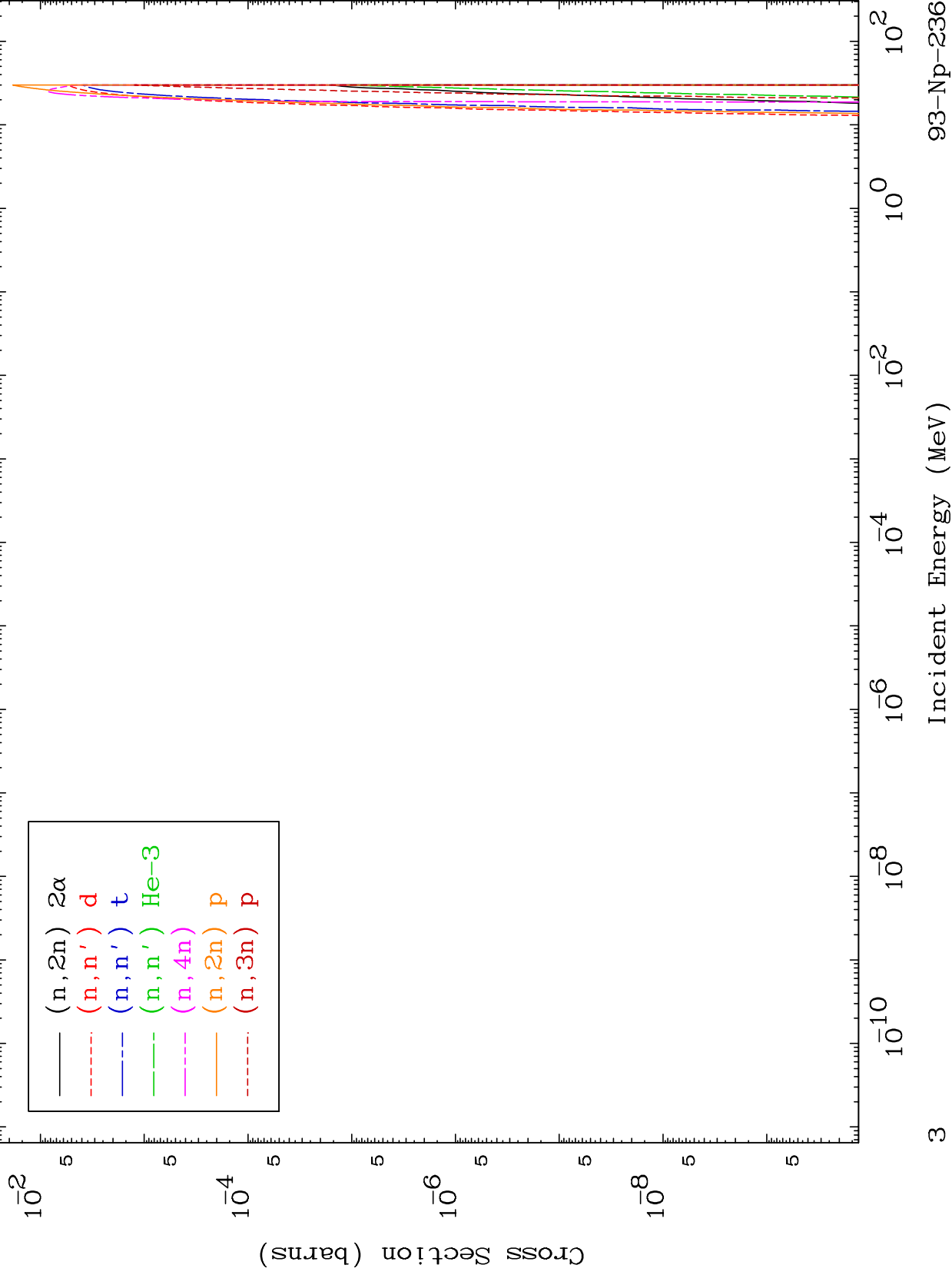
93-Np-236



MAT 9344

Neutron Production
293 Kelvin Cross Sections

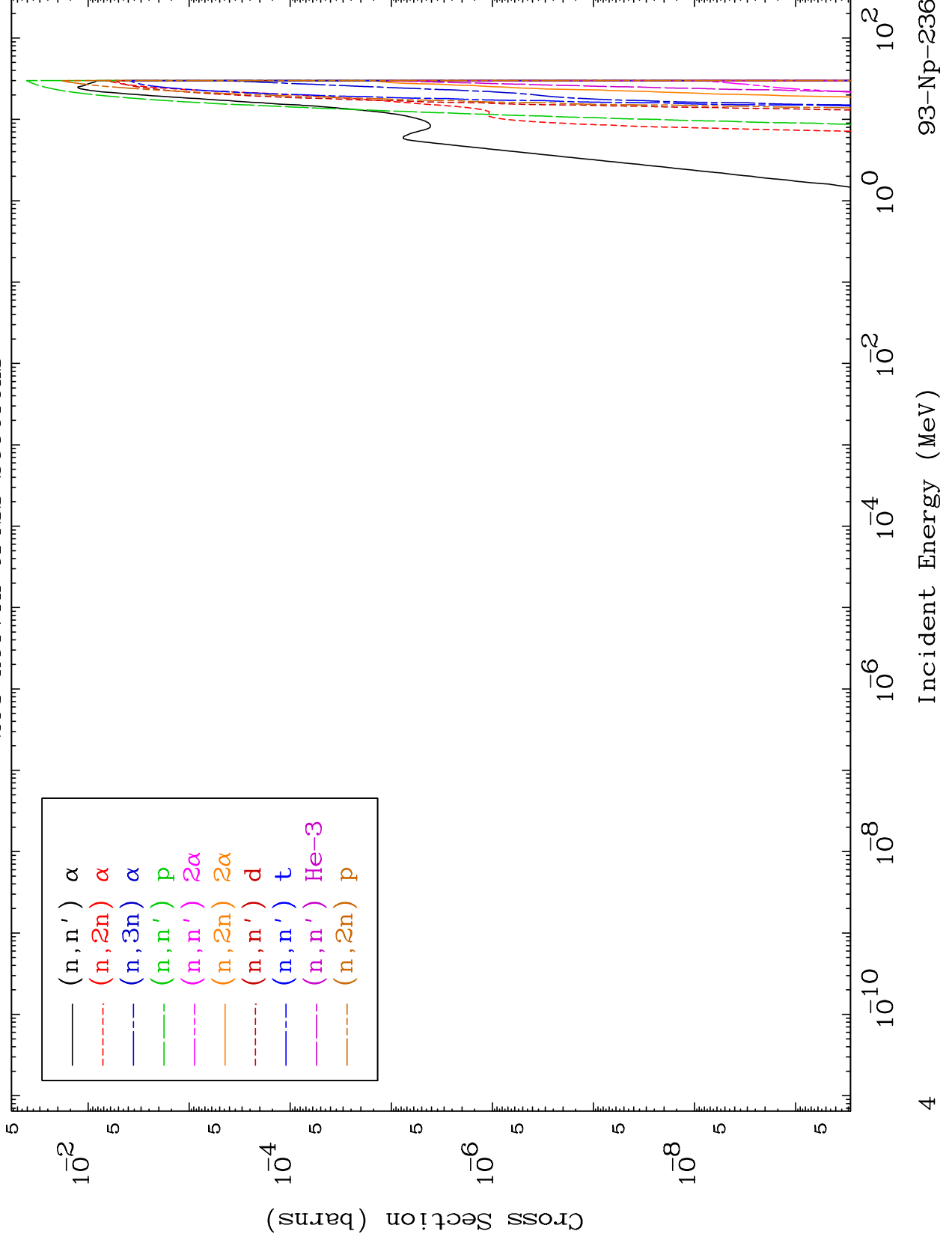
93-Np-236



MAT 9344

Charged Particle
293 Kelvin Cross Sections

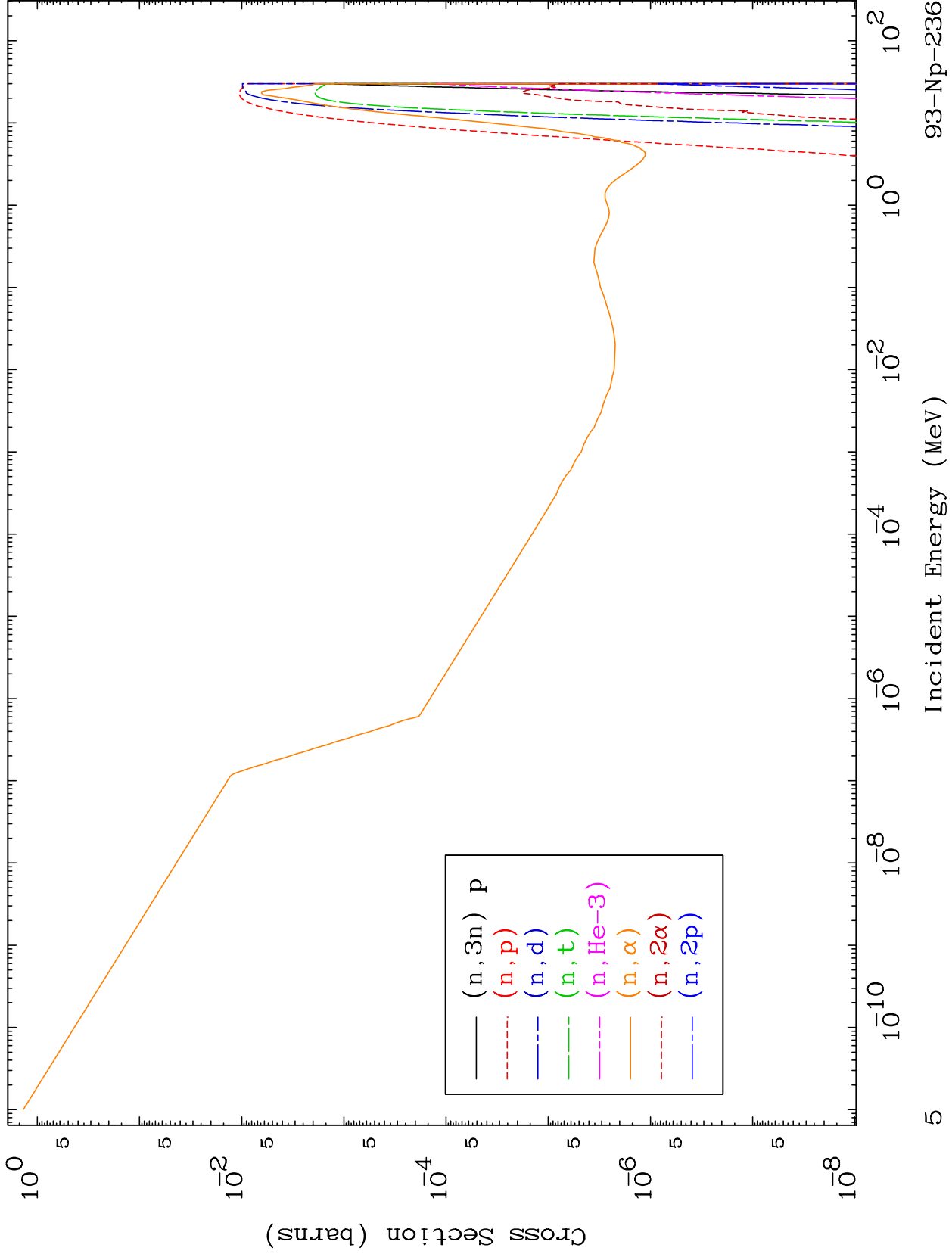
93-Np-236



MAT 9344

Charged Particle
293 Kelvin Cross Sections

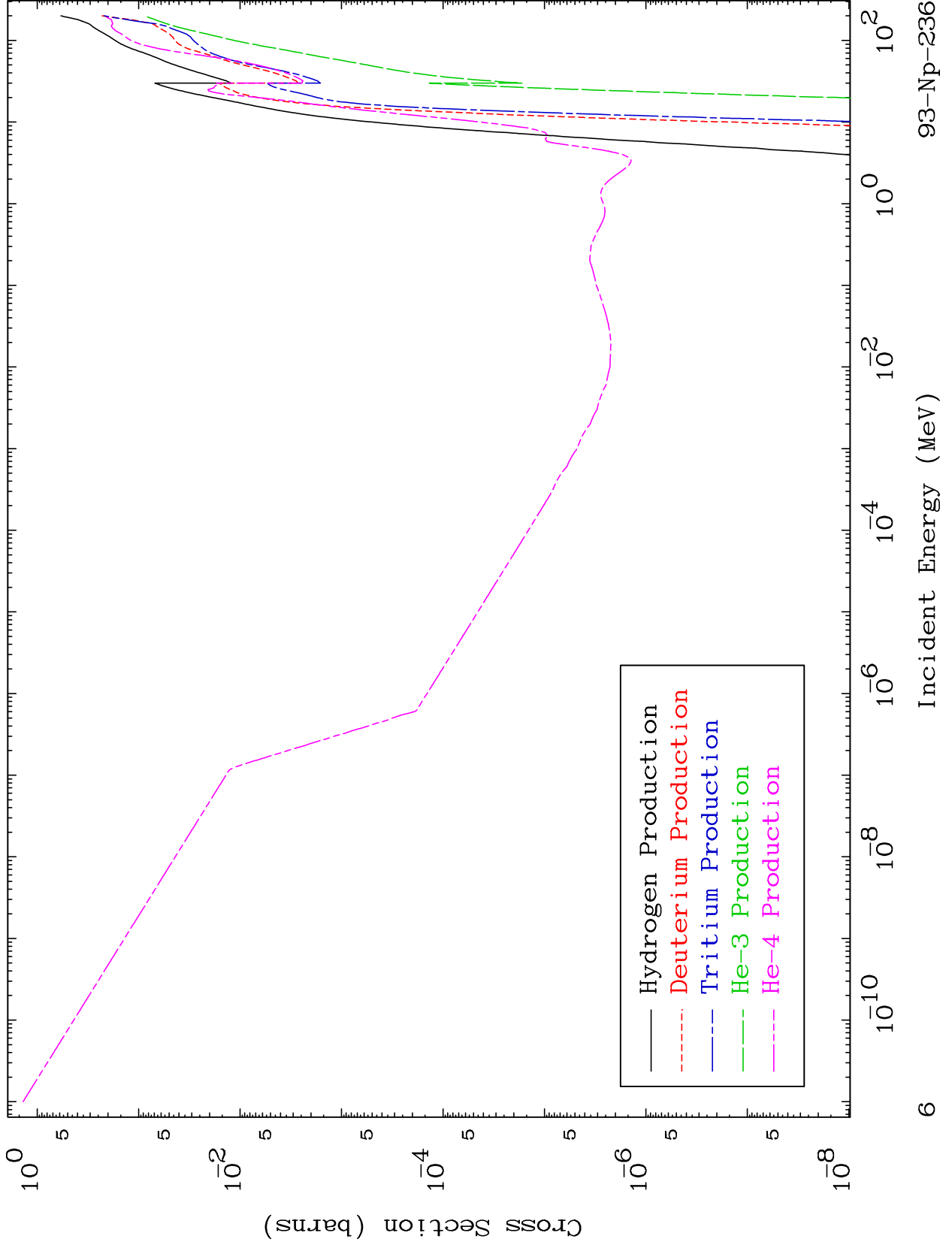
93-Np-236



MAT 9344

Particle Production
293 Kelvin Cross Sections

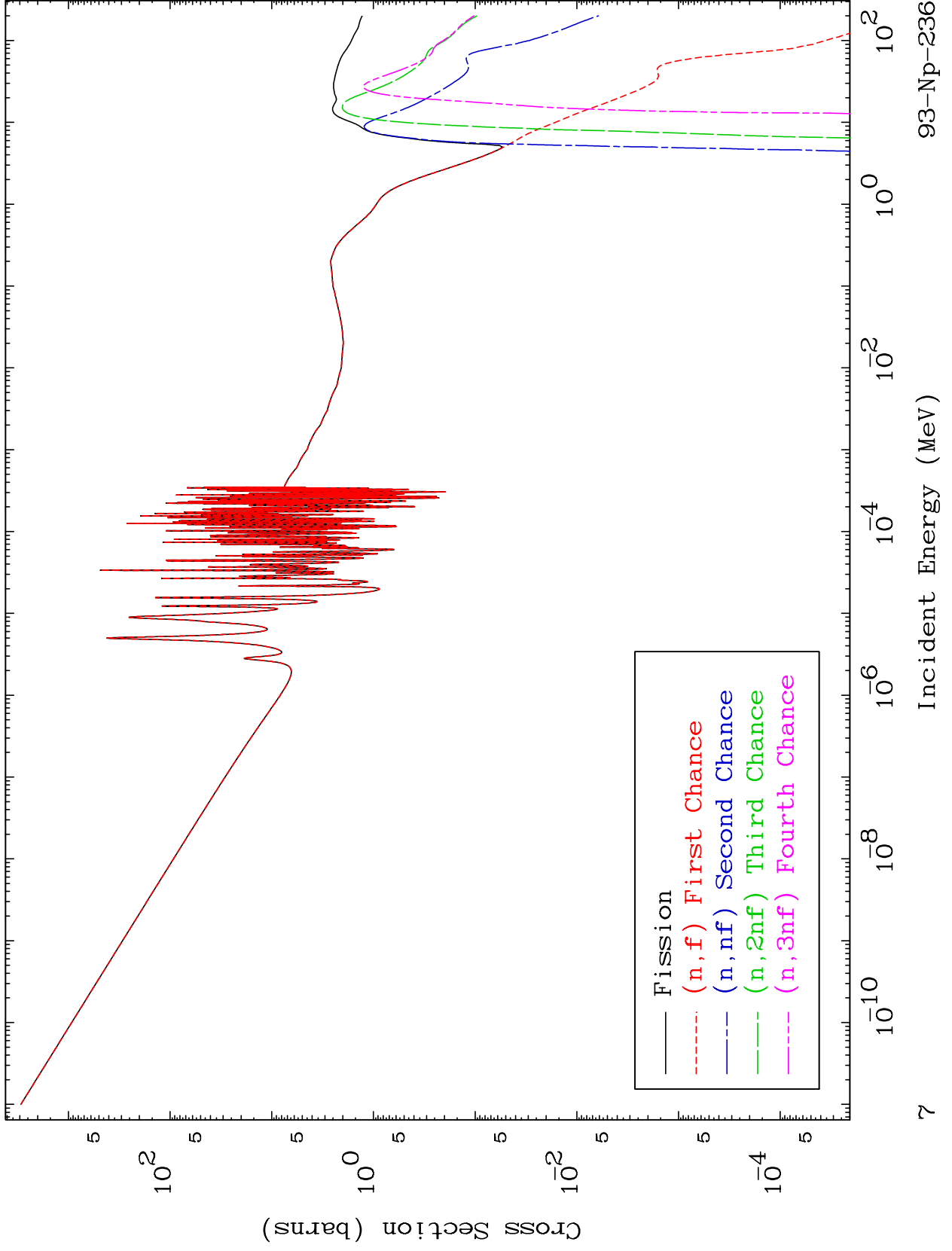
93-Np-236



MAT 9344

Fission
293 Kelvin Cross Sections

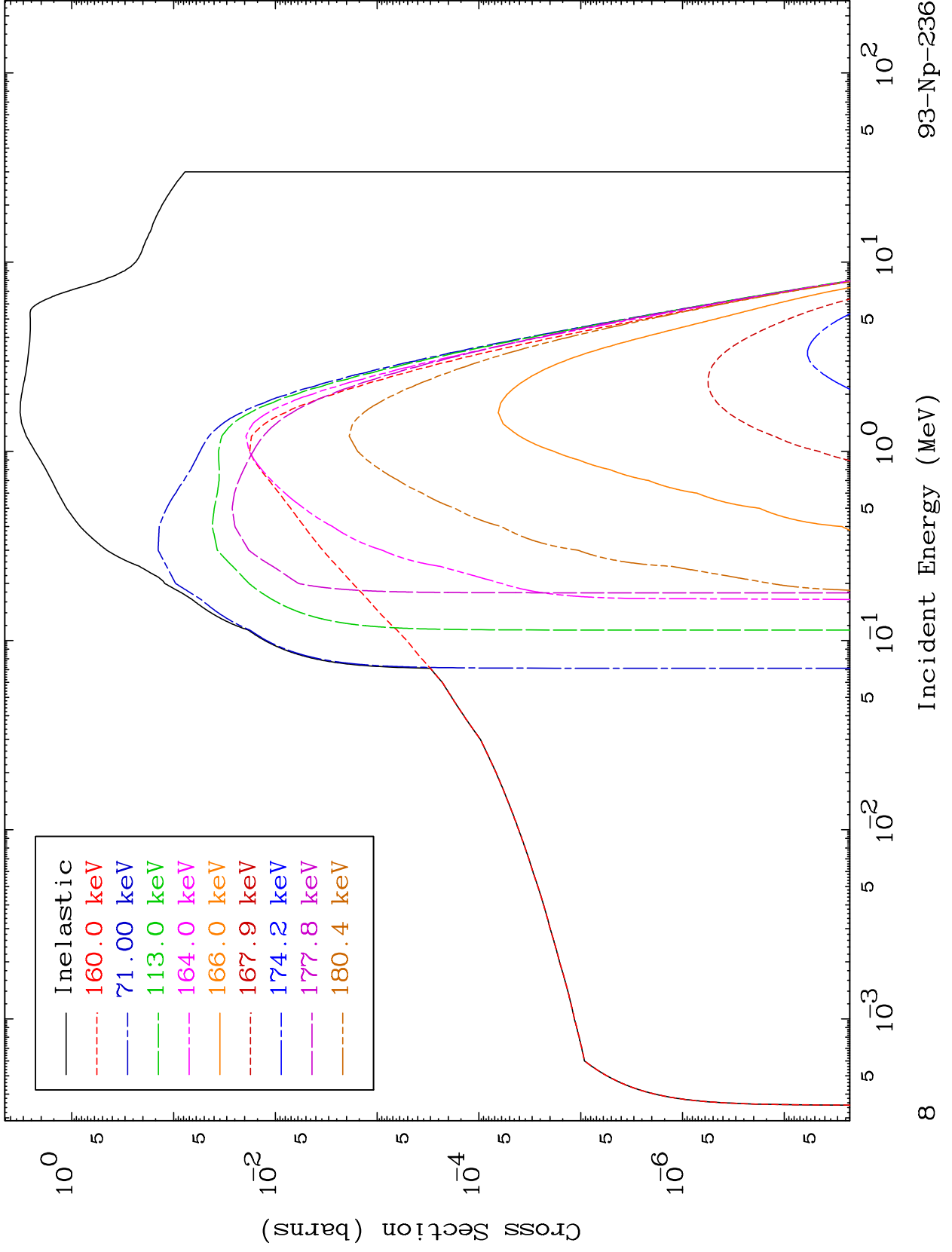
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MAT 9344

(n,n') Level
293 Kelvin Cross Sections

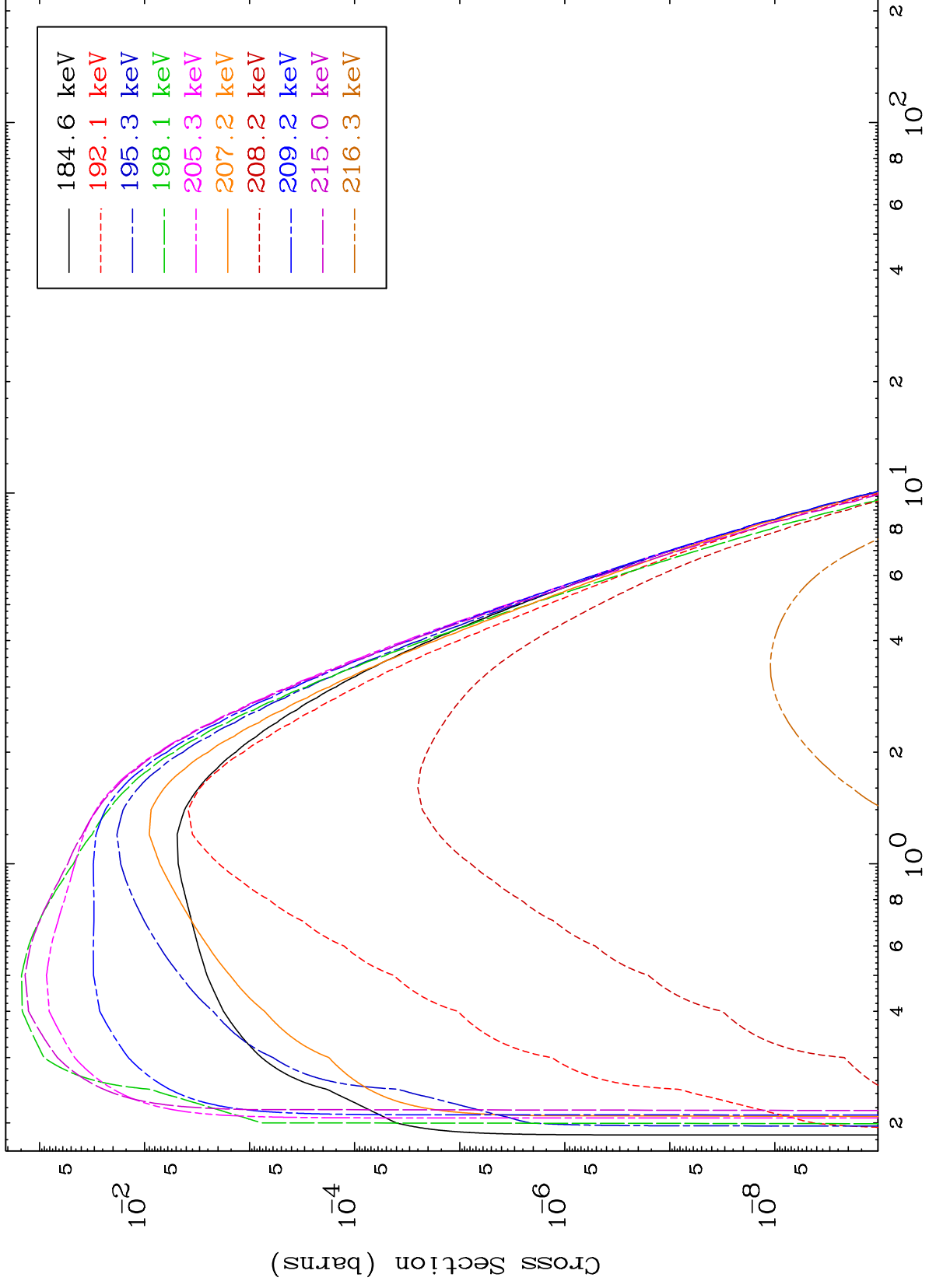
93-Np-236



MAT 9344

(n,n') Level
293 Kelvin Cross Sections

93-Np-236



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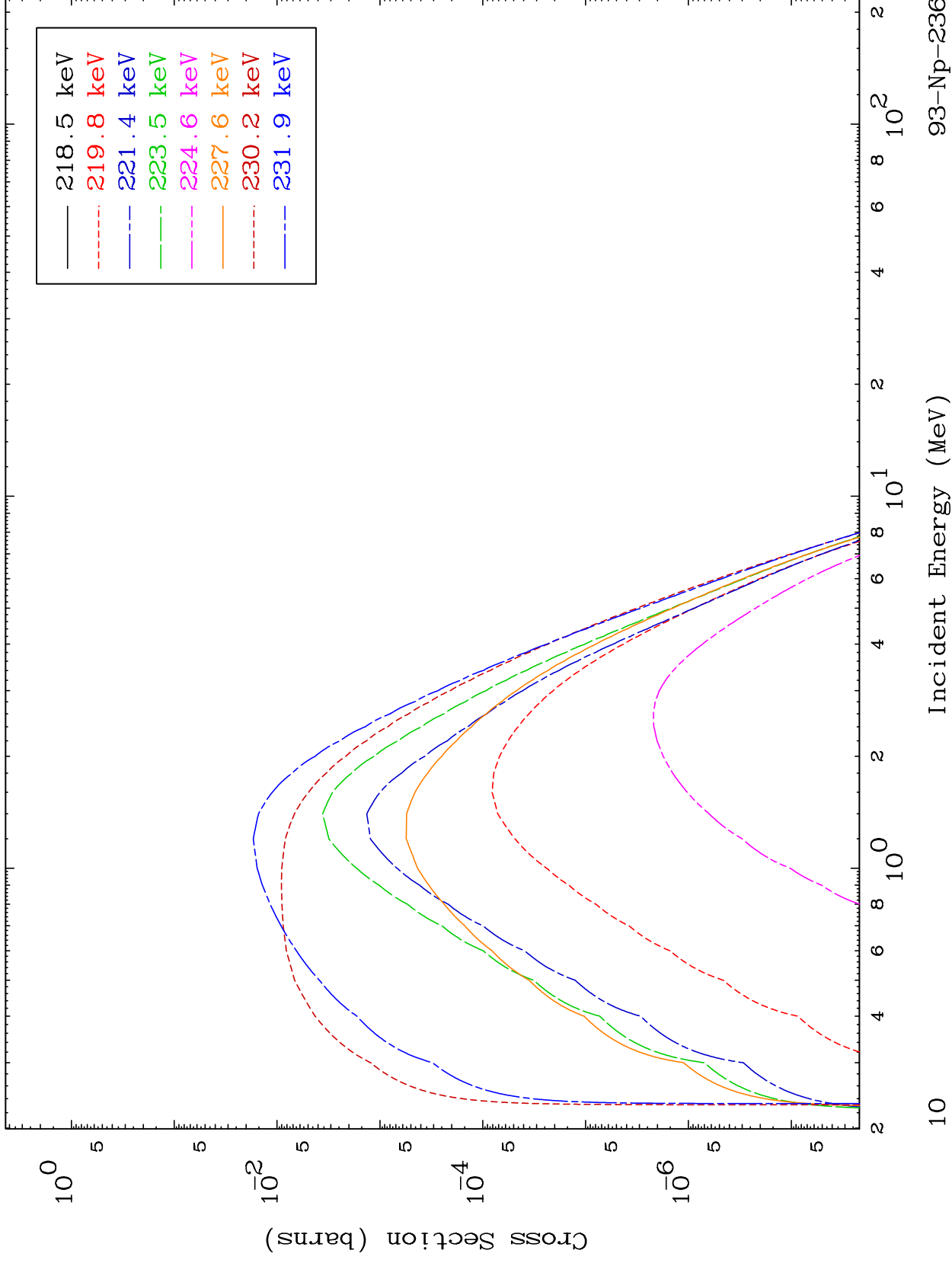
Incident Energy (MeV)

93-Np-236

MAT 9344

(n,n') Level
293 Kelvin Cross Sections

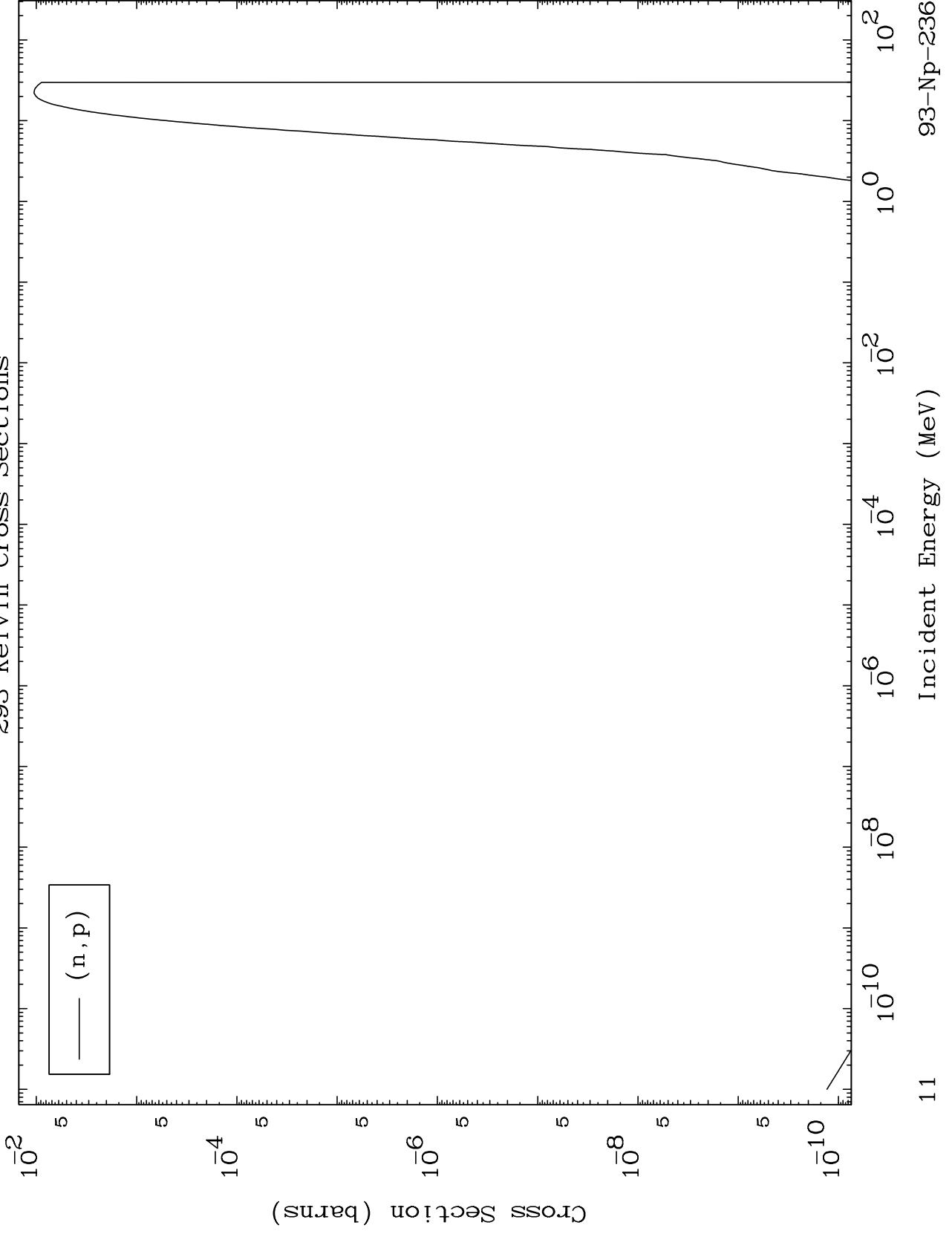
93-Np-236



MAT 9344

(n,p) Levels
293 Kelvin Cross Sections

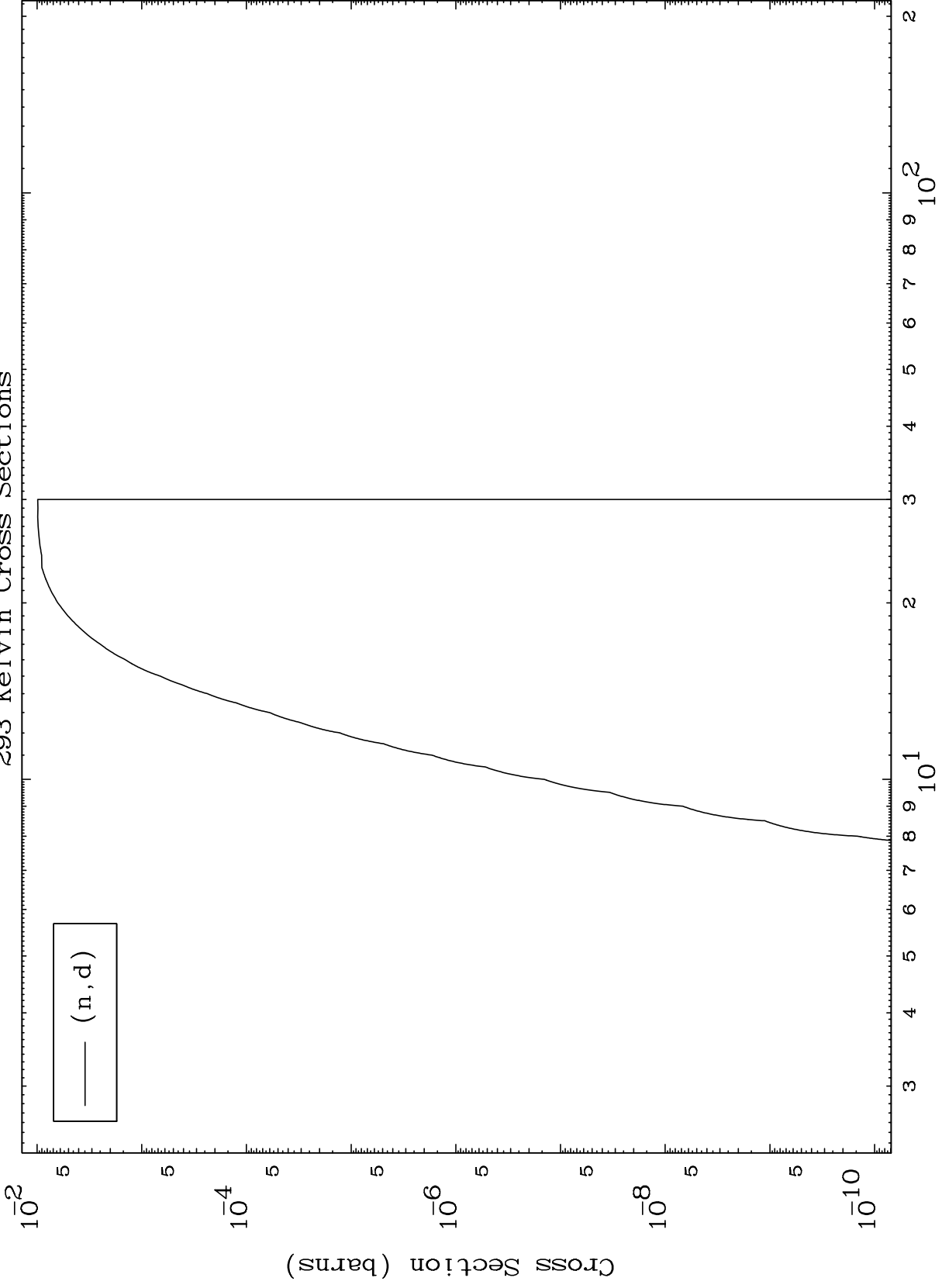
93-Np-236



MAT 9344

(n,d) Levels
293 Kelvin Cross Sections

93-Np-236



12

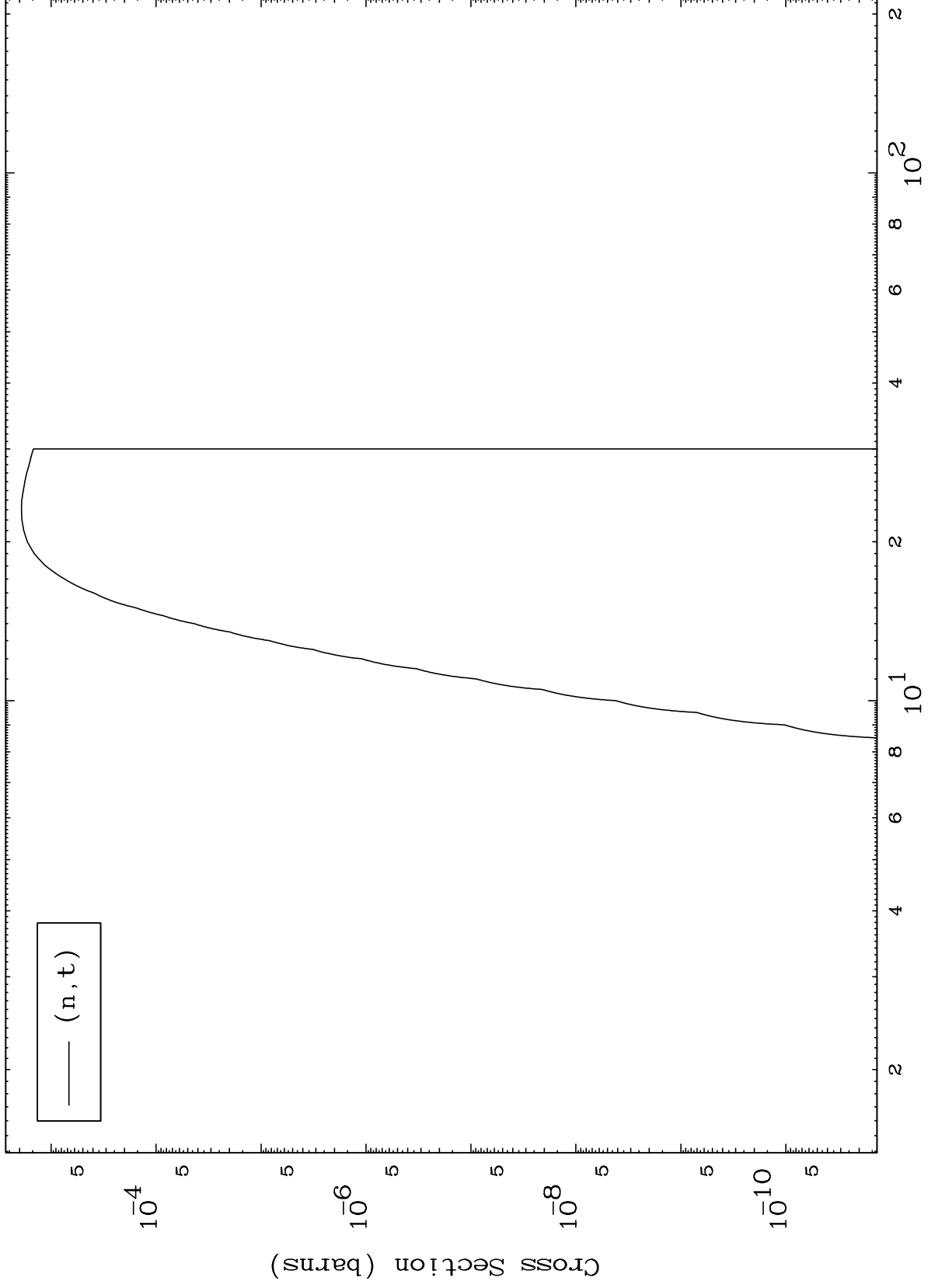
Incident Energy (MeV)

93-Np-236

MAT 9344

(n,t) Levels
293 Kelvin Cross Sections

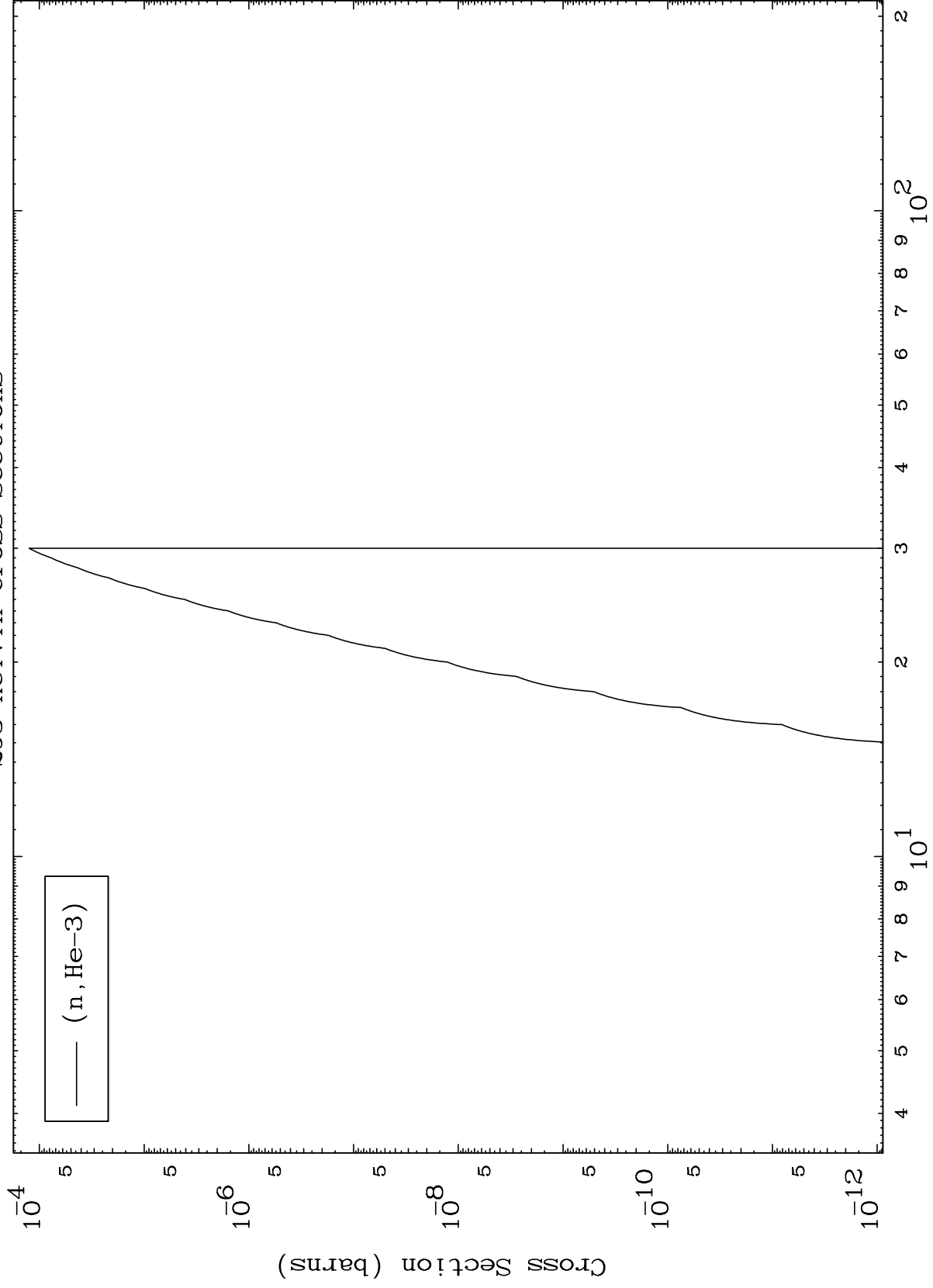
93-Np-236



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(n,He3) Levels
293 Kelvin Cross Sections

93-Np-236



14

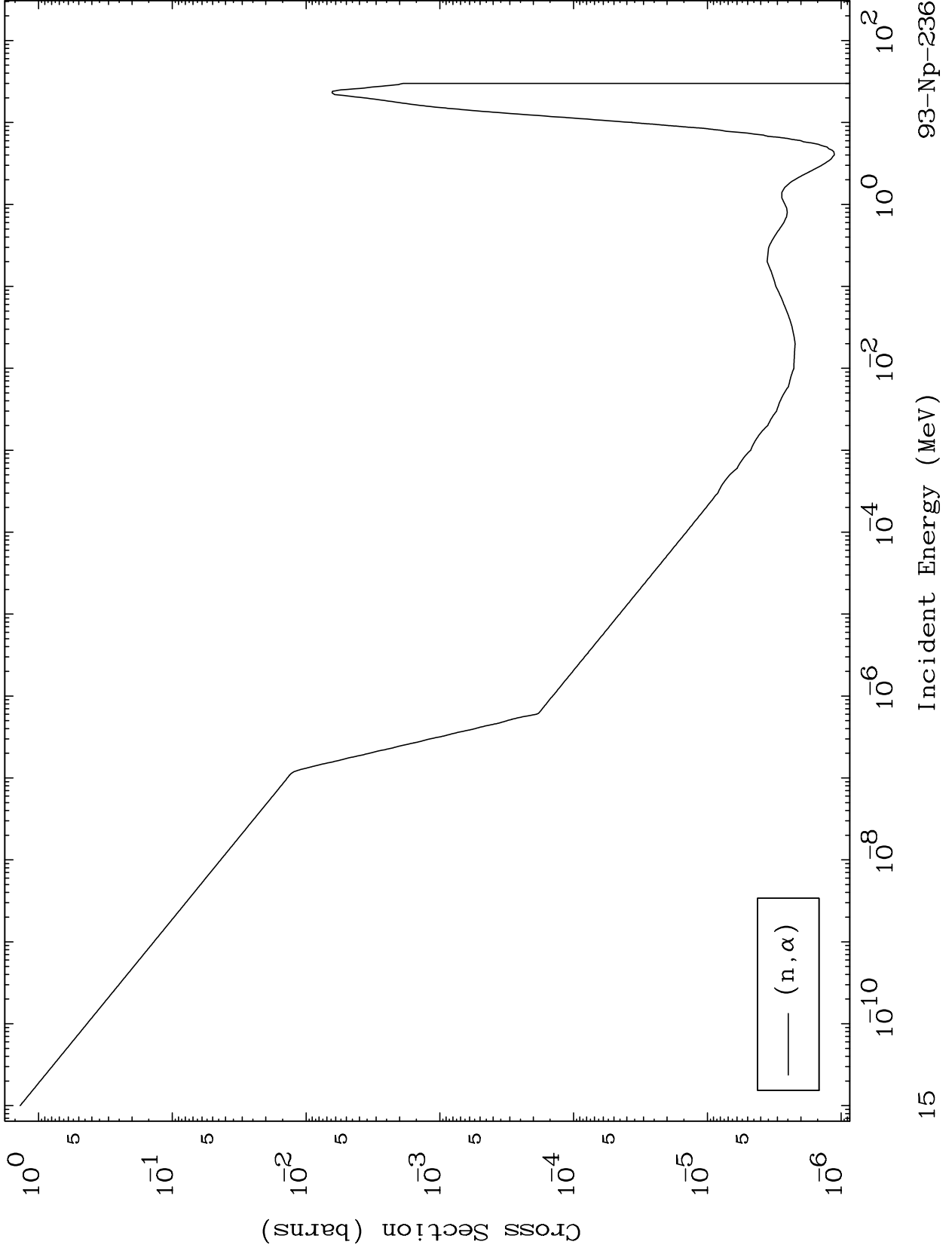
Incident Energy (MeV)

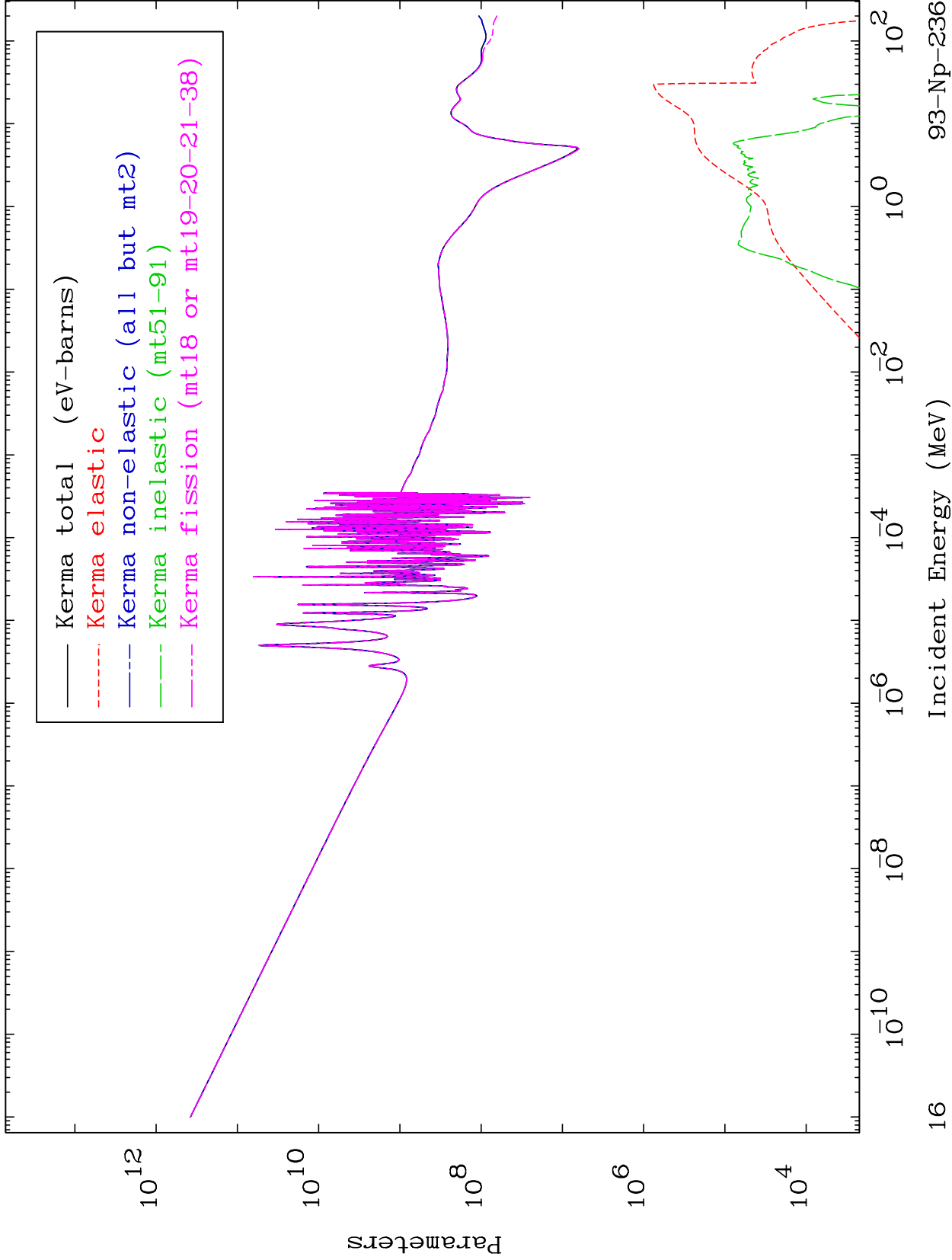
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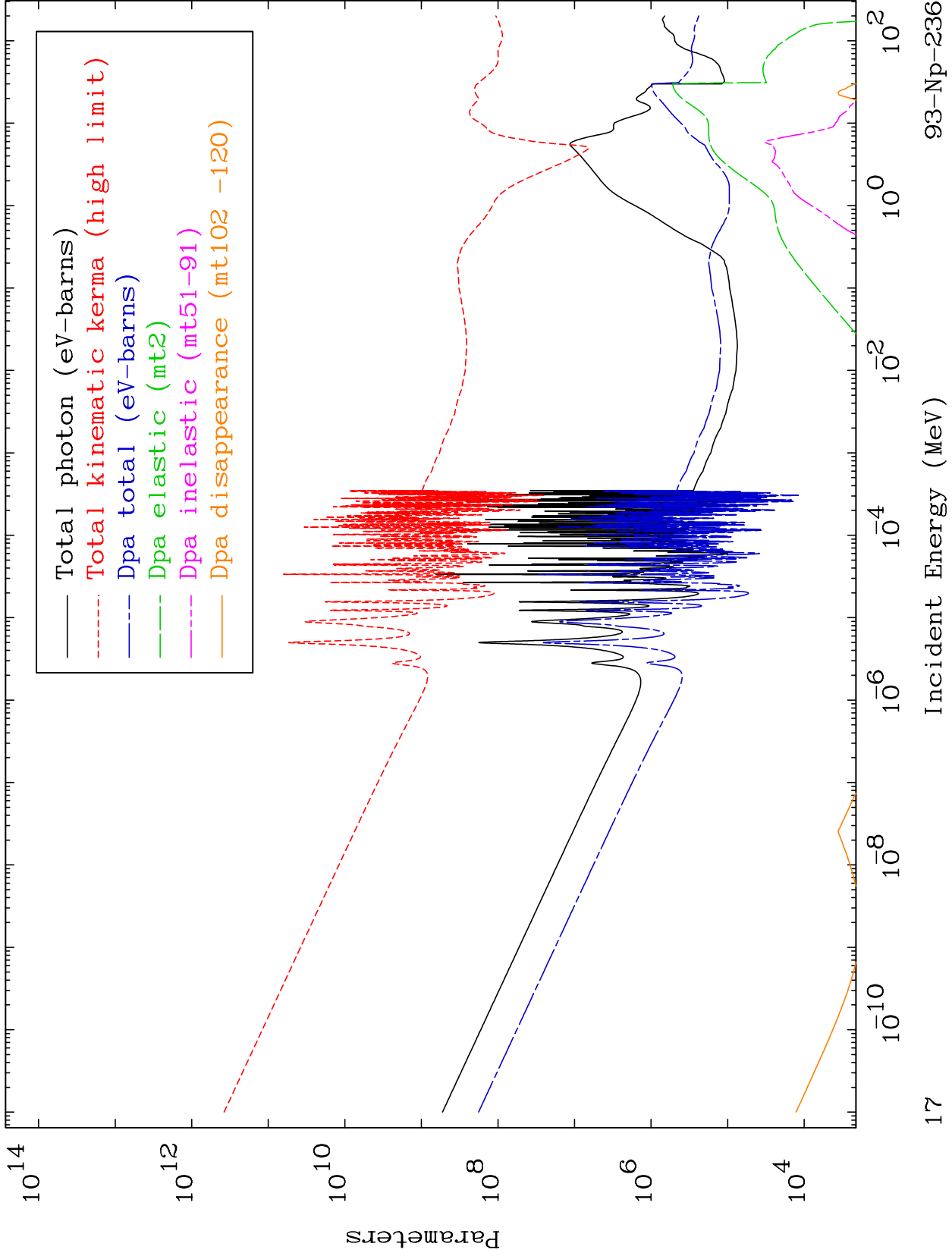
MAT 9344

(n, α) Levels
293 Kelvin Cross Sections

93-Np-236



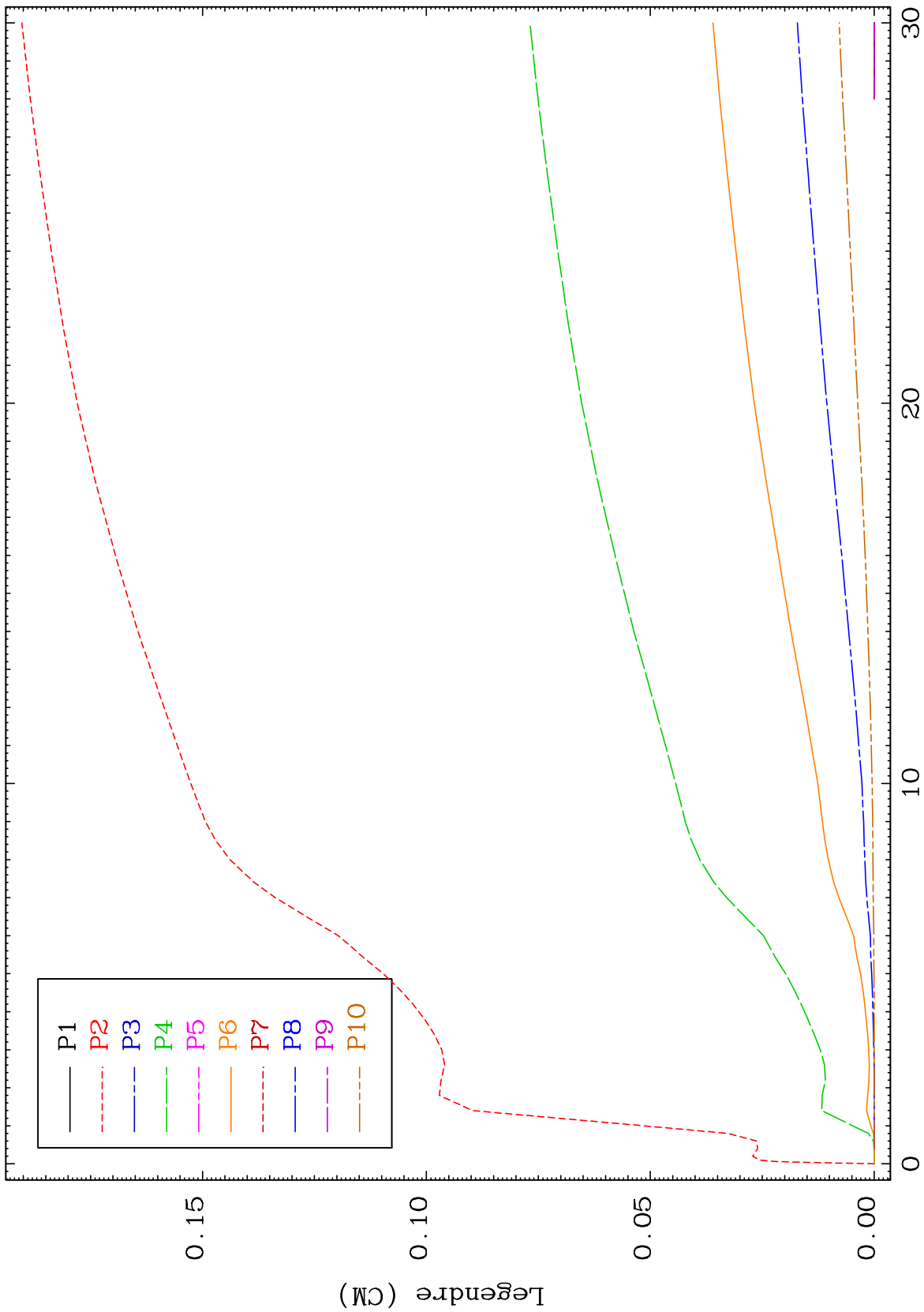




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Elastic Legendre Coefficients

93-Np-236



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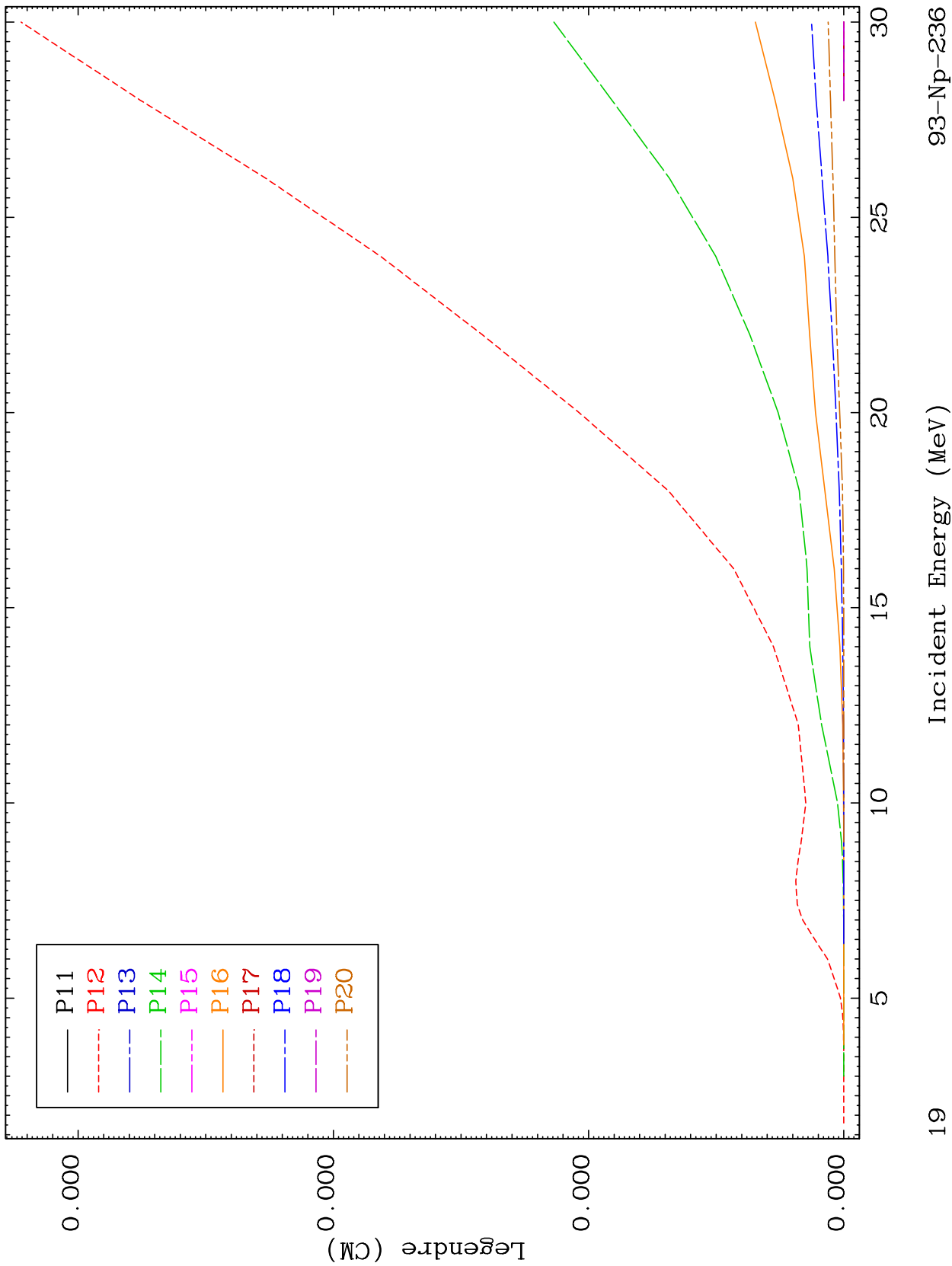
Incident Energy (MeV)

93-Np-236

MAT 9344

Elastic Legendre Coefficients

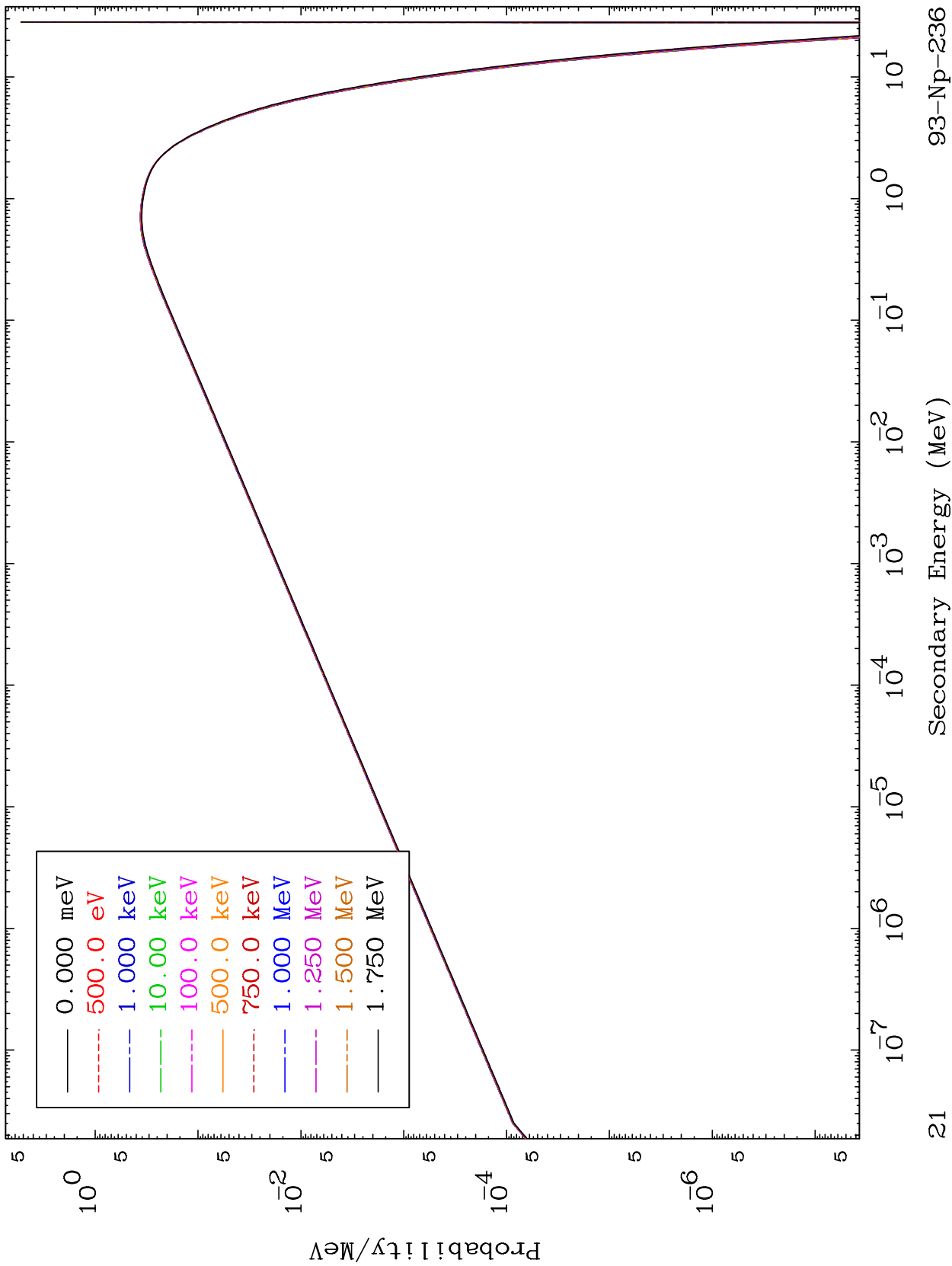
93-Np-236



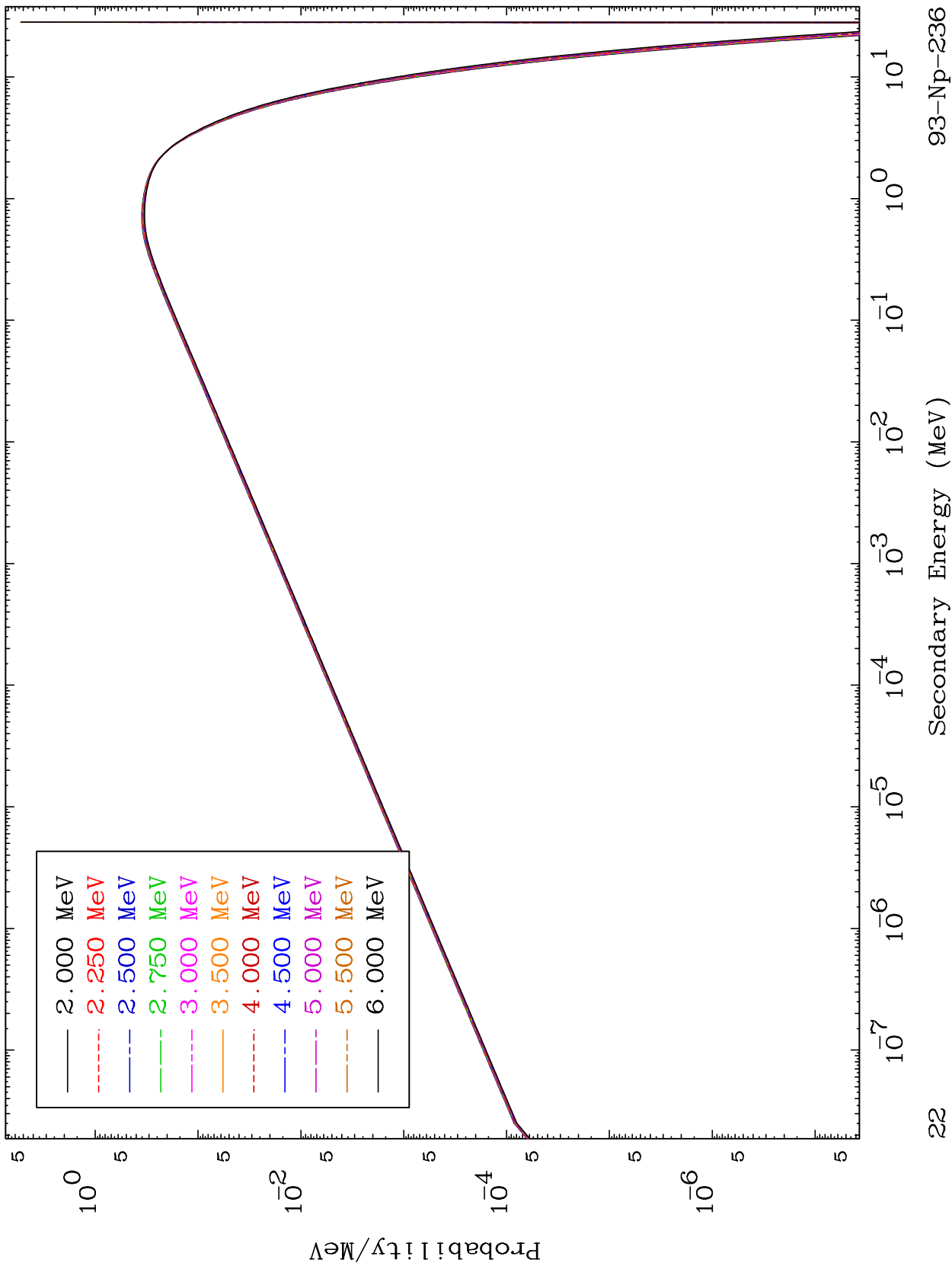
19

93-Np-236

Fission Energy Distribution



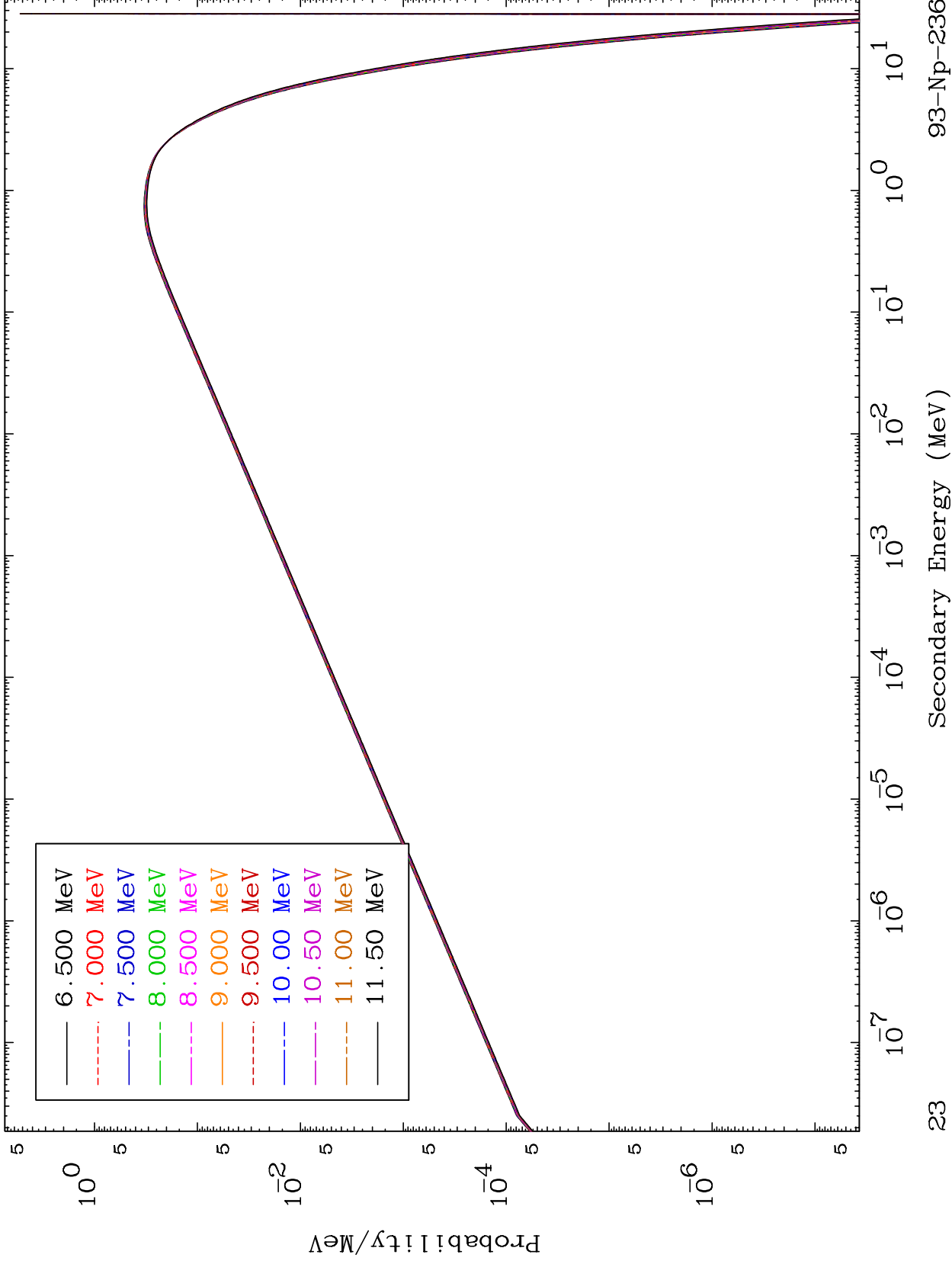
Fission Energy Distribution



MAT 9344

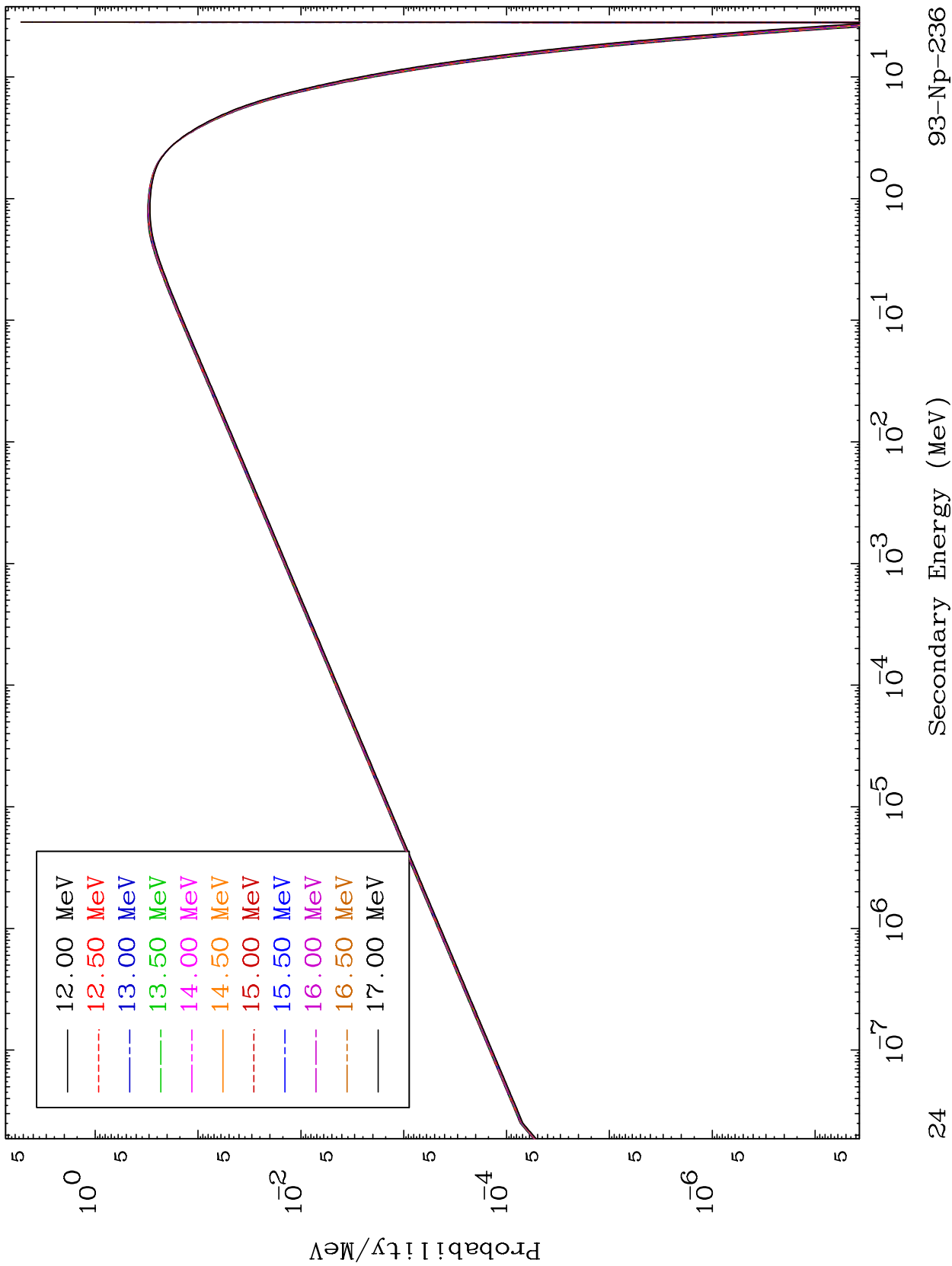
Fission Energy Distribution

93-Np-236

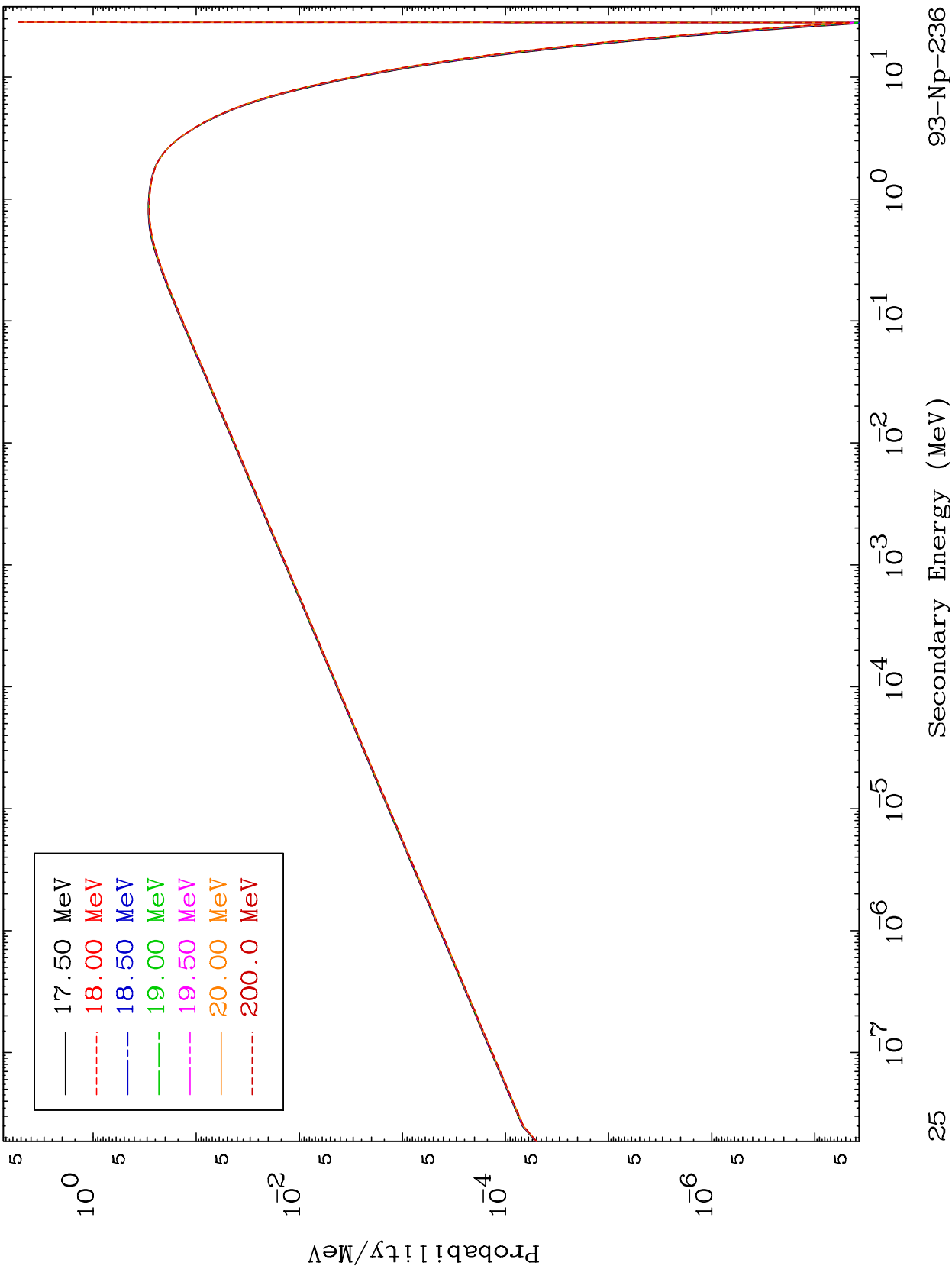


93-Np-236

Fission
Energy Distribution



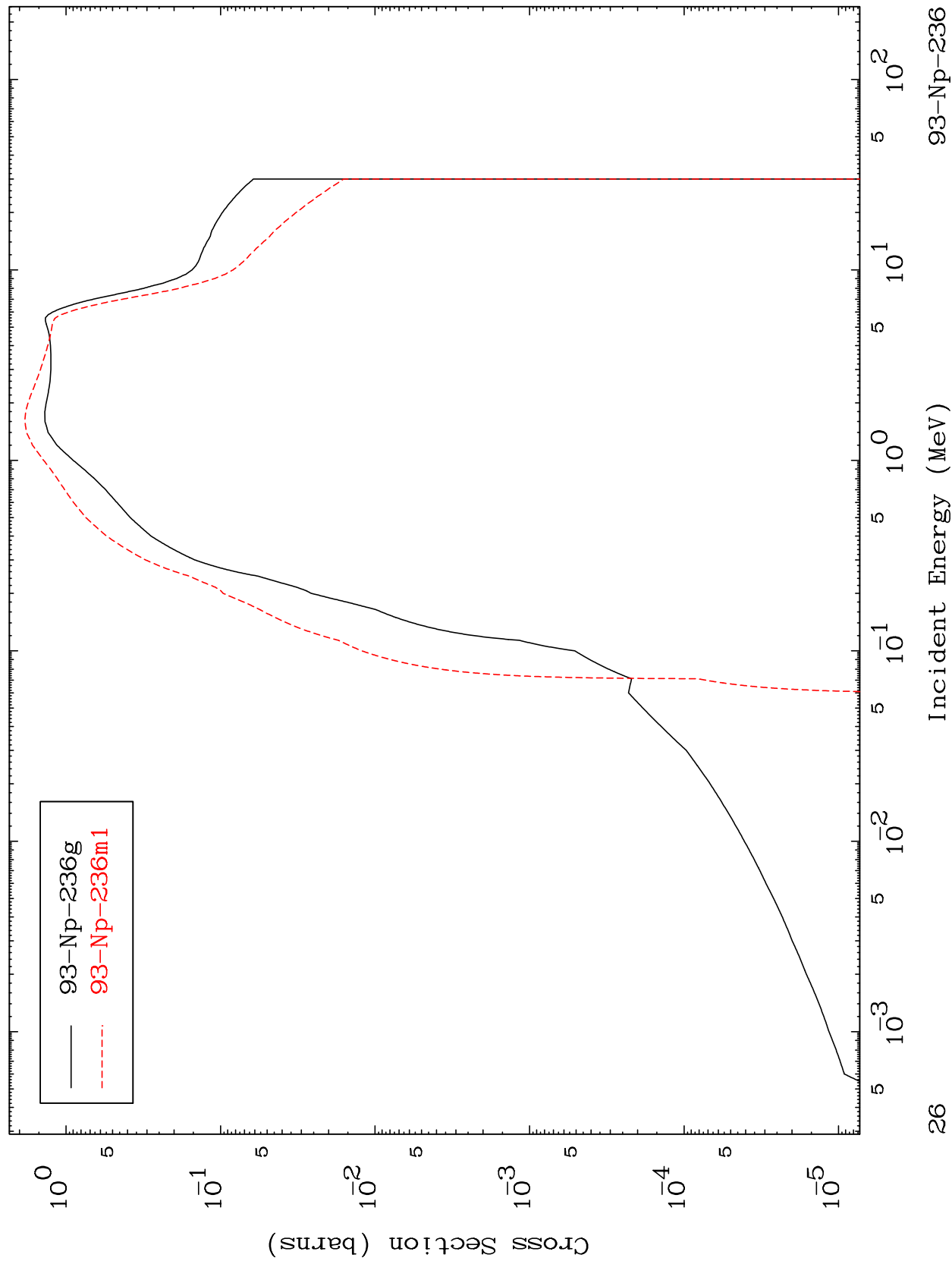
Fission
Energy Distribution



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93-Np-236

Inelastic
Radionuclide Production Cross Section



26

93-Np-236

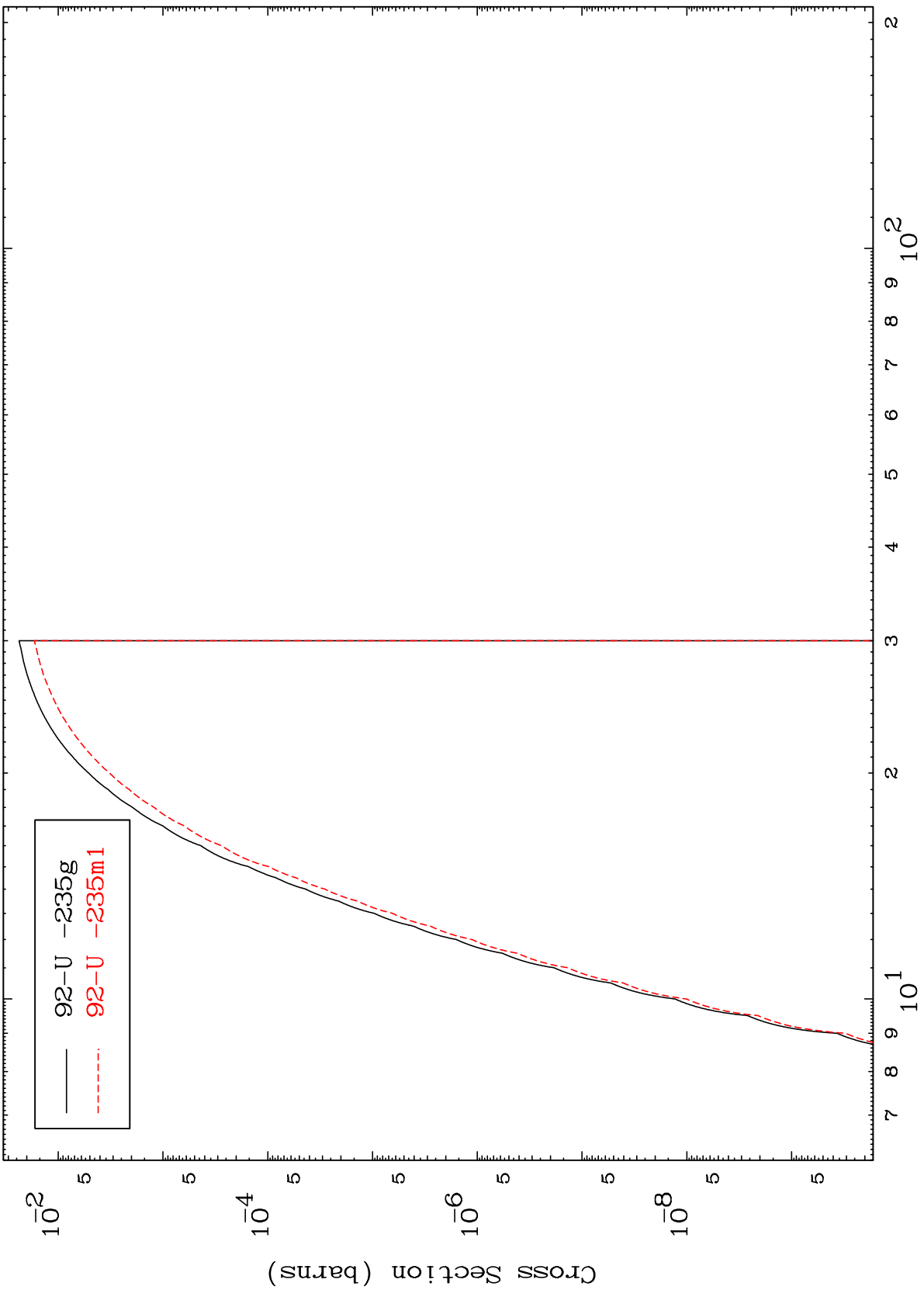
Incident Energy (MeV)

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(n,n') p

93-Np-236

Radionuclide Production Cross Section



27

Incident Energy (MeV)

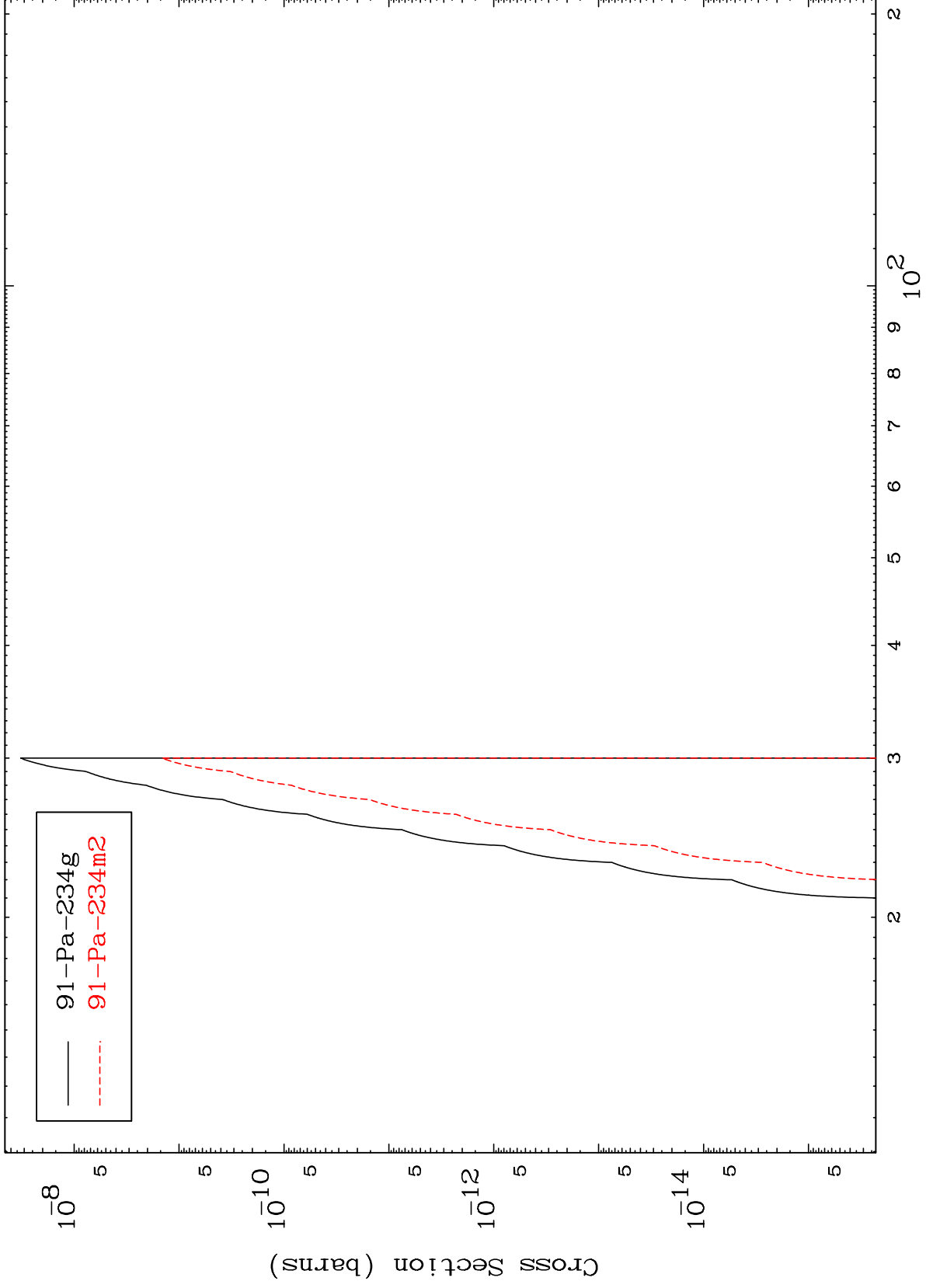
93-Np-236

MAT 9344

(n,2n) p

93-Np-236

Radionuclide Production Cross Section



28

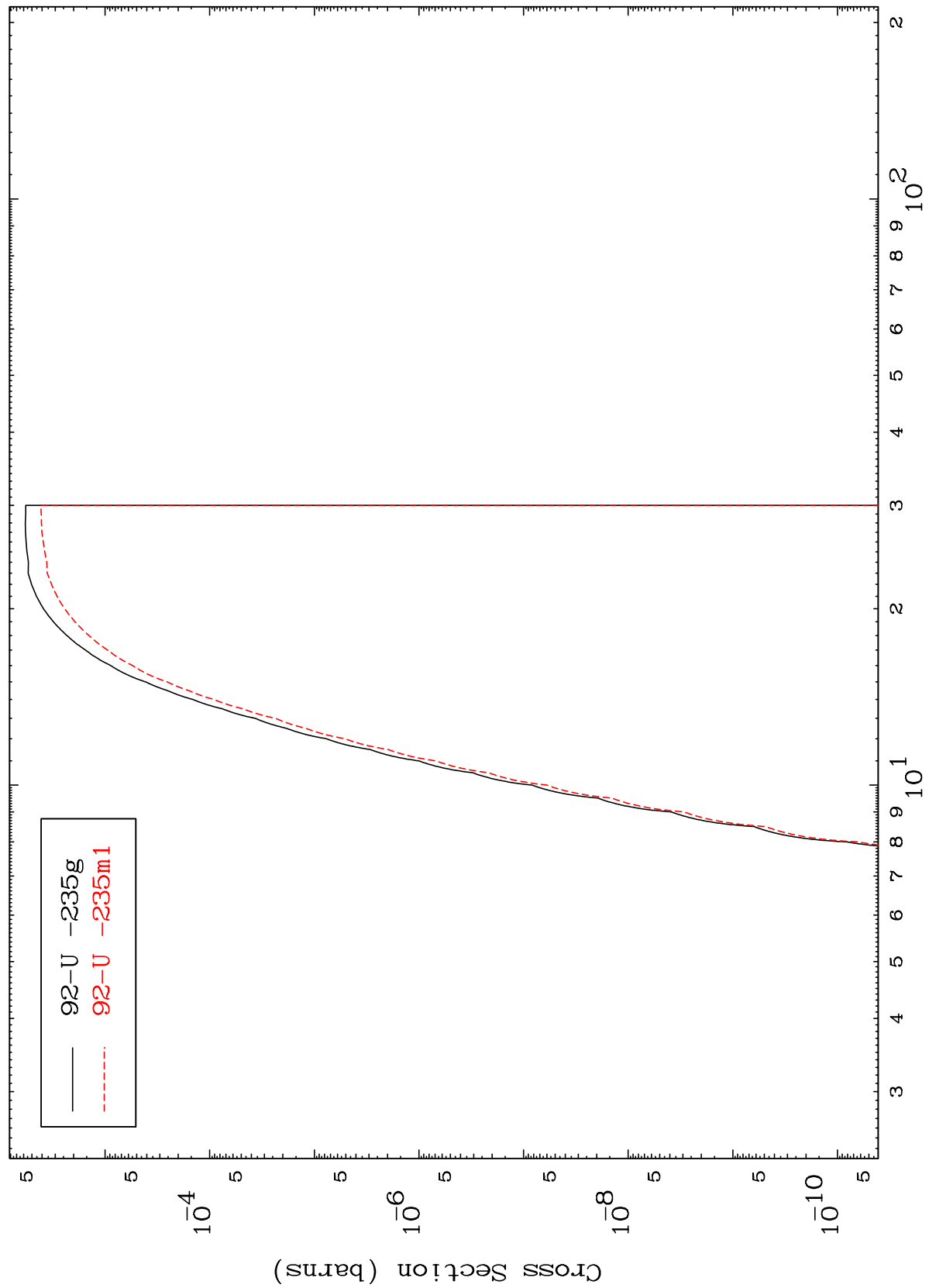
Incident Energy (MeV)

93-Np-236

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93-Np-236

(n,d)
Radionuclide Production Cross Section



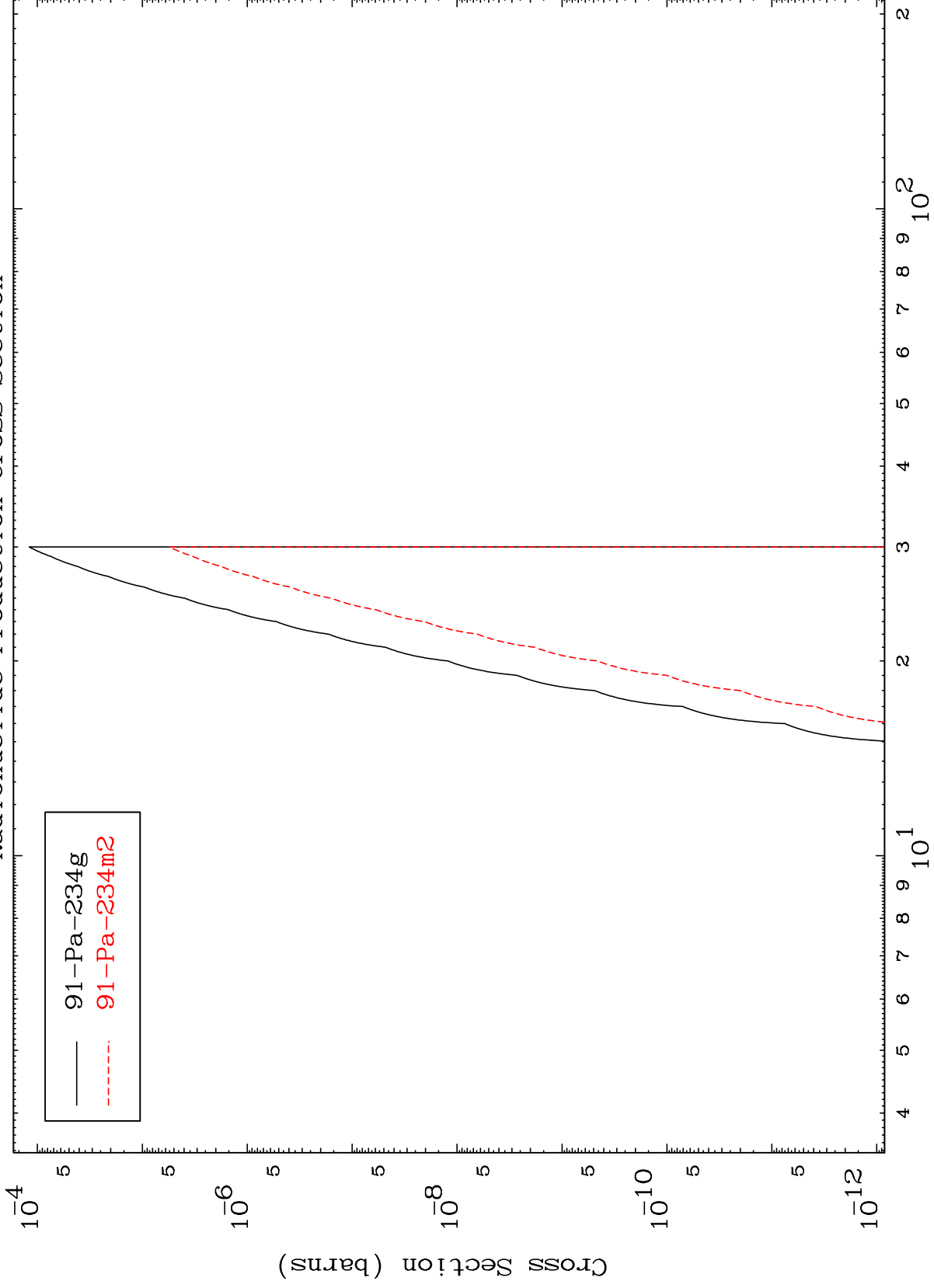
— 92-U -235g
- - - 92-U -235m1

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(n,He-3)

93-Np-236

Radionuclide Production Cross Section



91-Pa-234g
91-Pa-234m2

30

Incident Energy (MeV)

93-Np-236