

Program EVALPLOT
(Version 2015-2)

by

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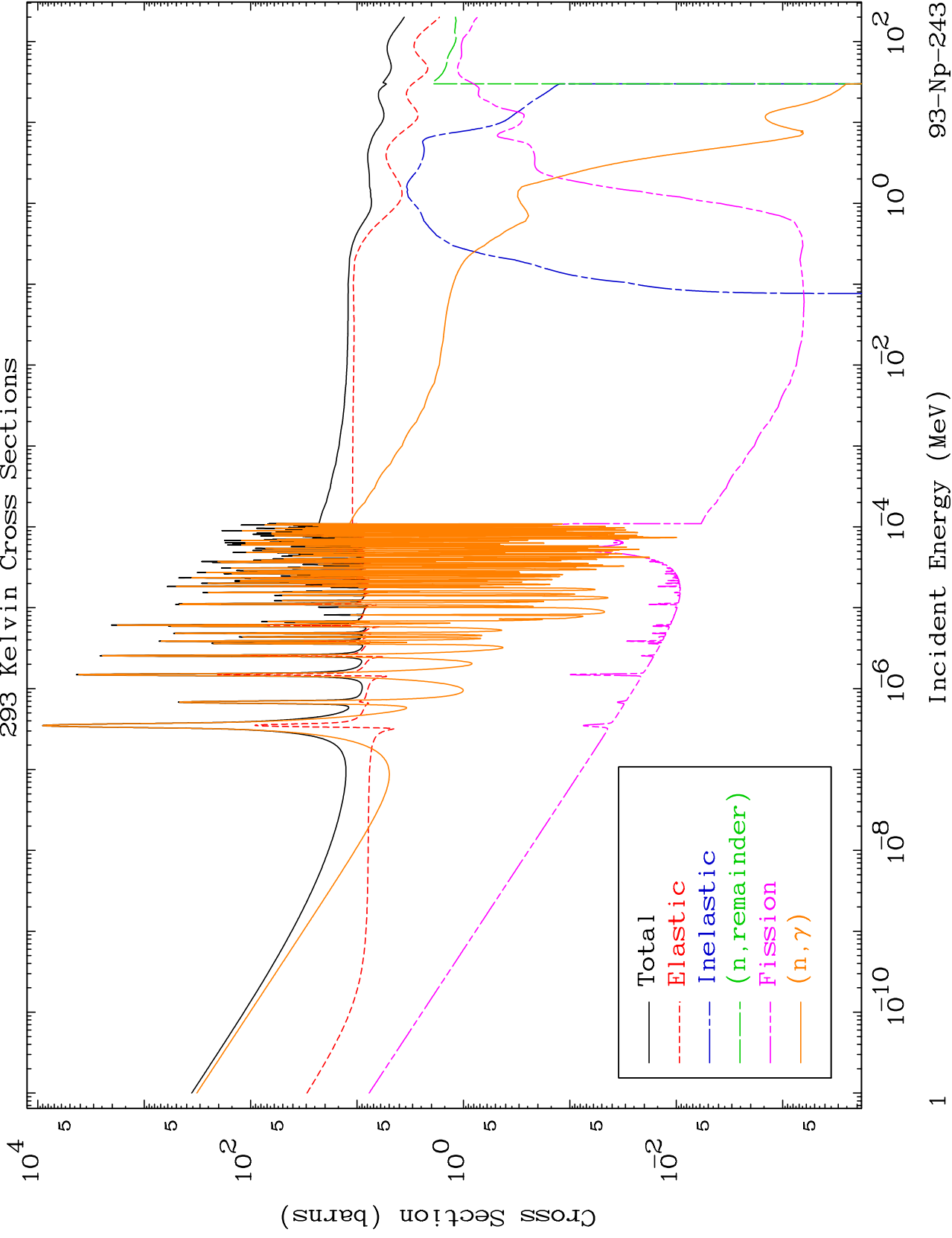
Press Mouse Button to Start

MAT 9364

Major

293 Kelvin Cross Sections

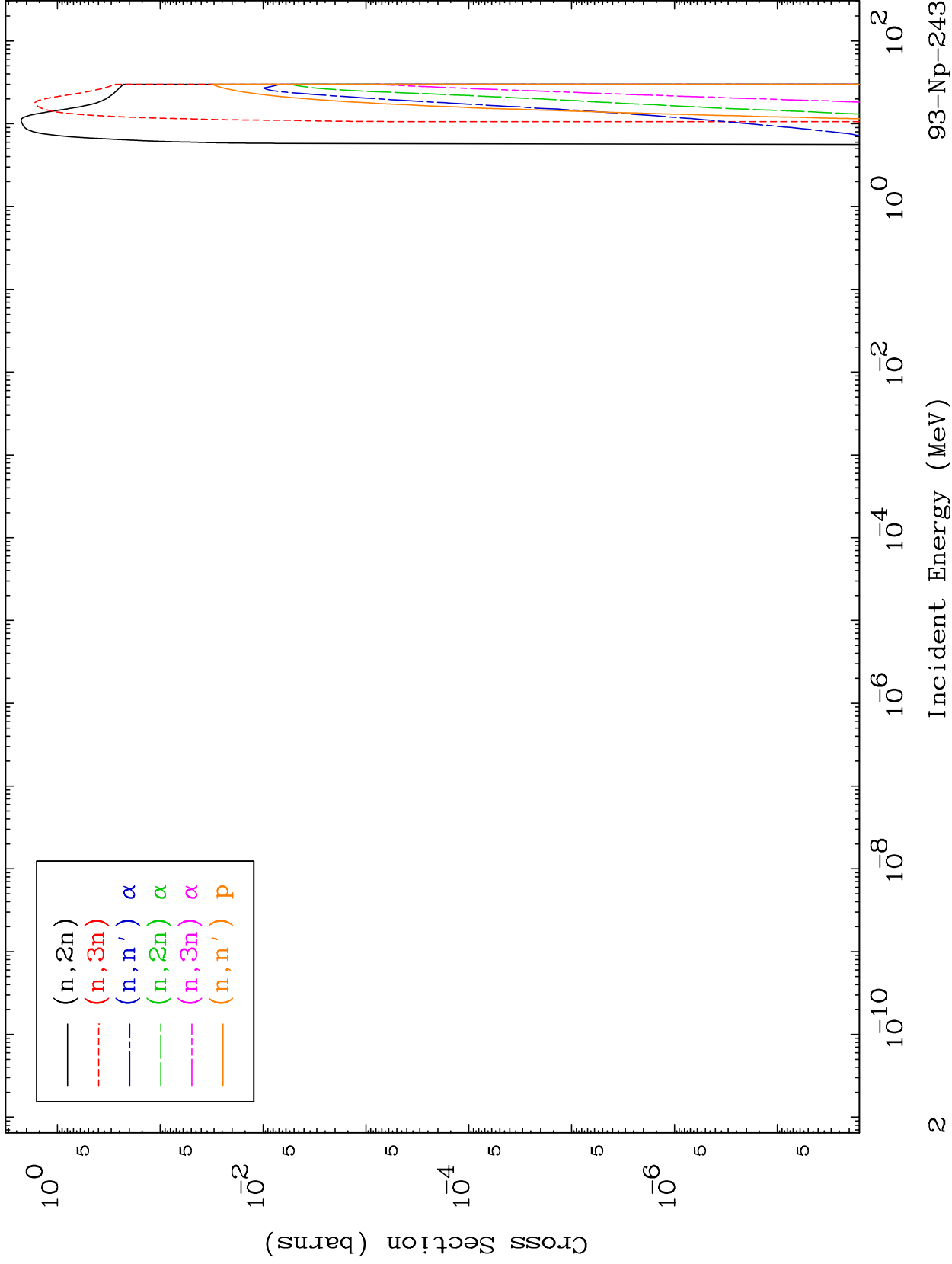
93-Np-243

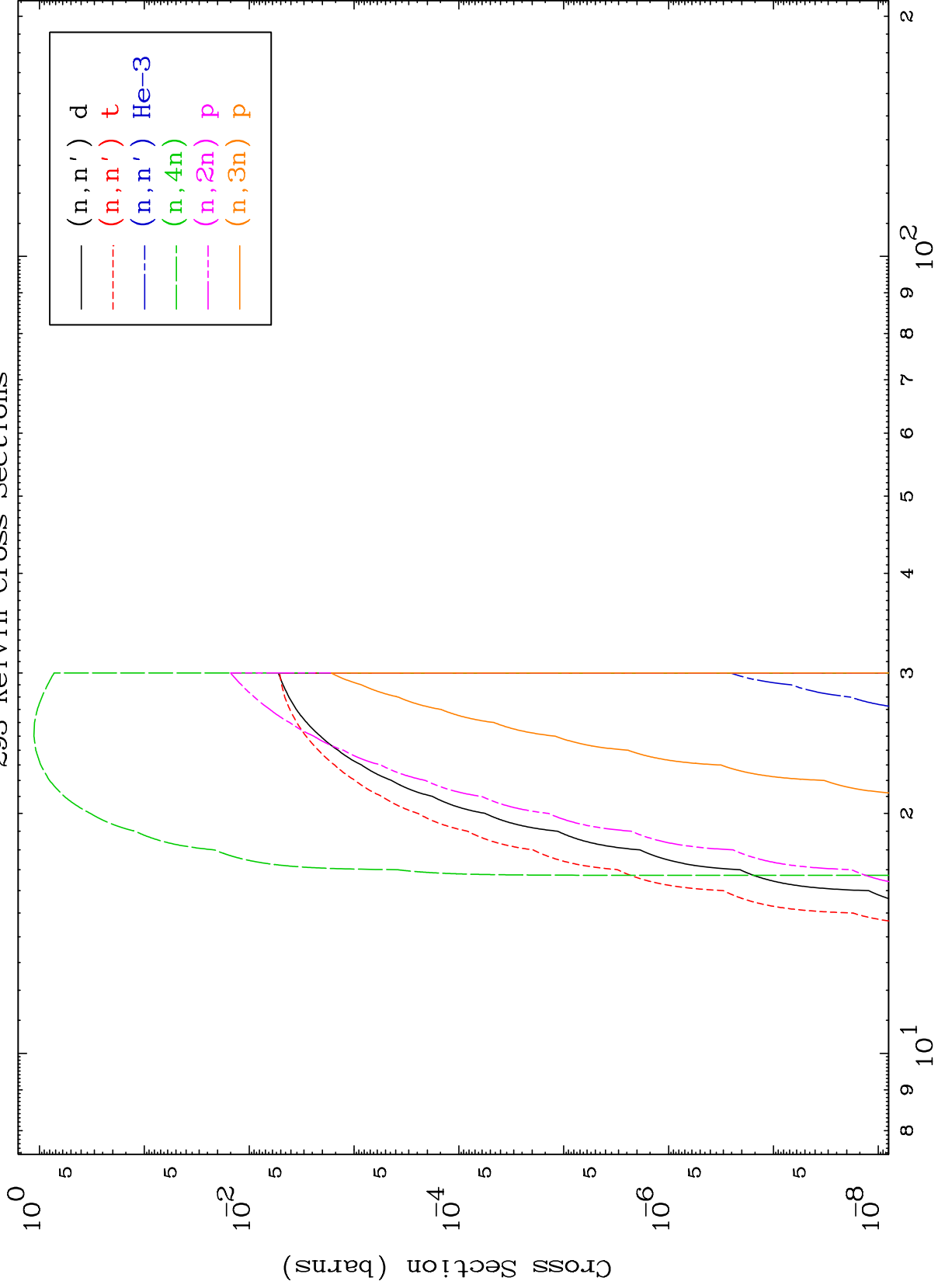


MAT 9364

Neutron Production
293 Kelvin Cross Sections

93-Np-243

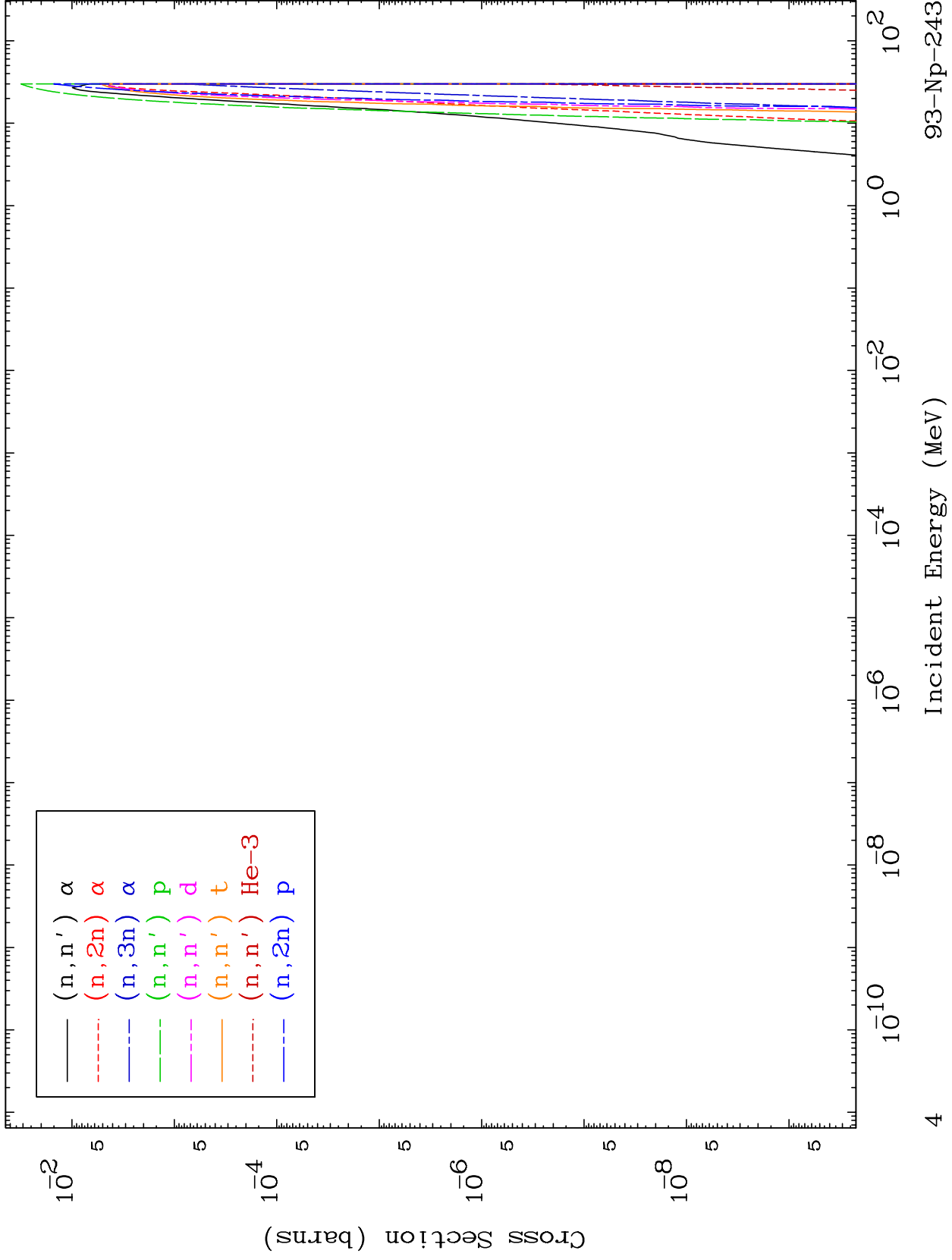




MAT 9364

Charged Particle
293 Kelvin Cross Sections

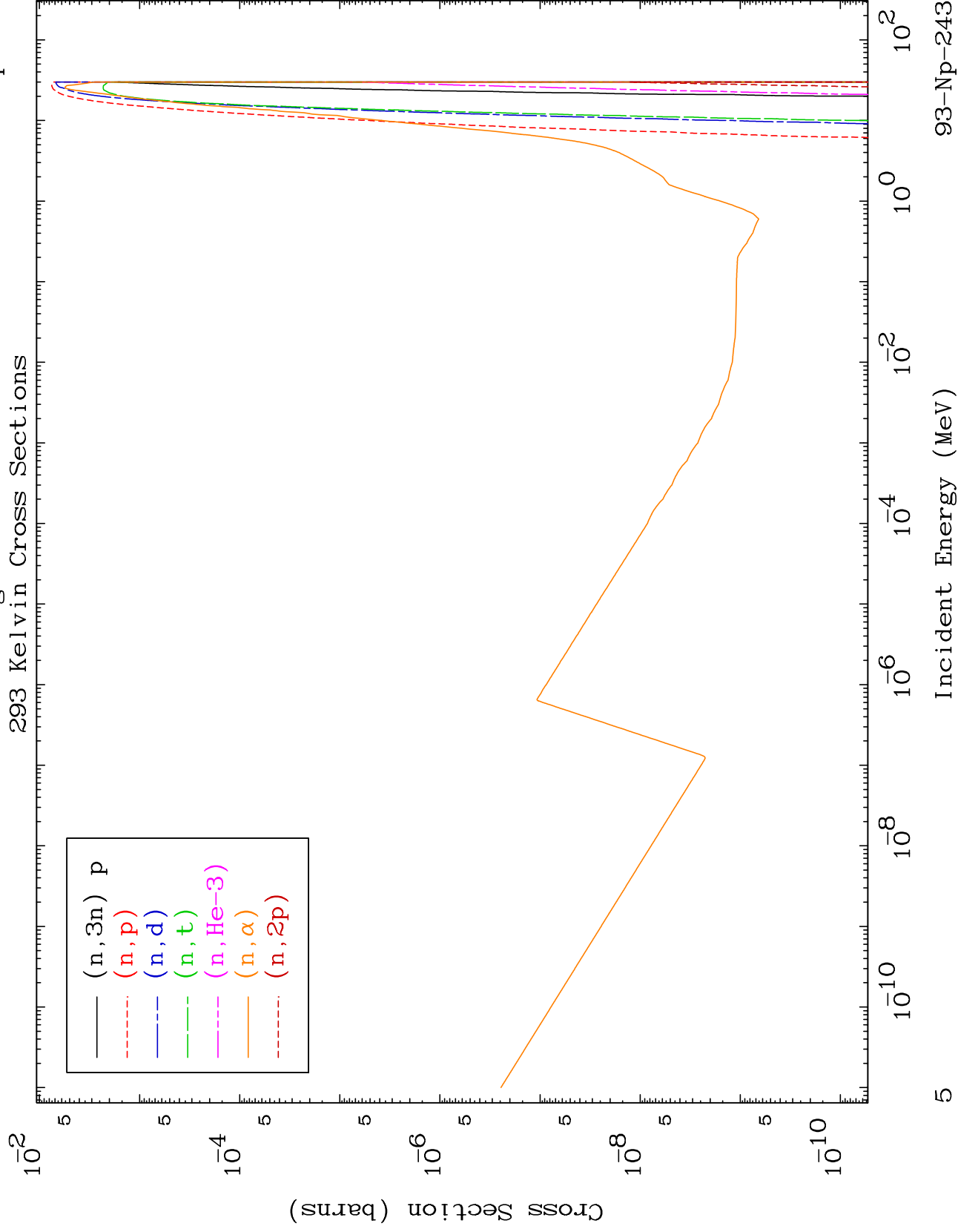
93-Np-243



MAT 9364

Charged Particle
293 Kelvin Cross Sections

93-Np-243



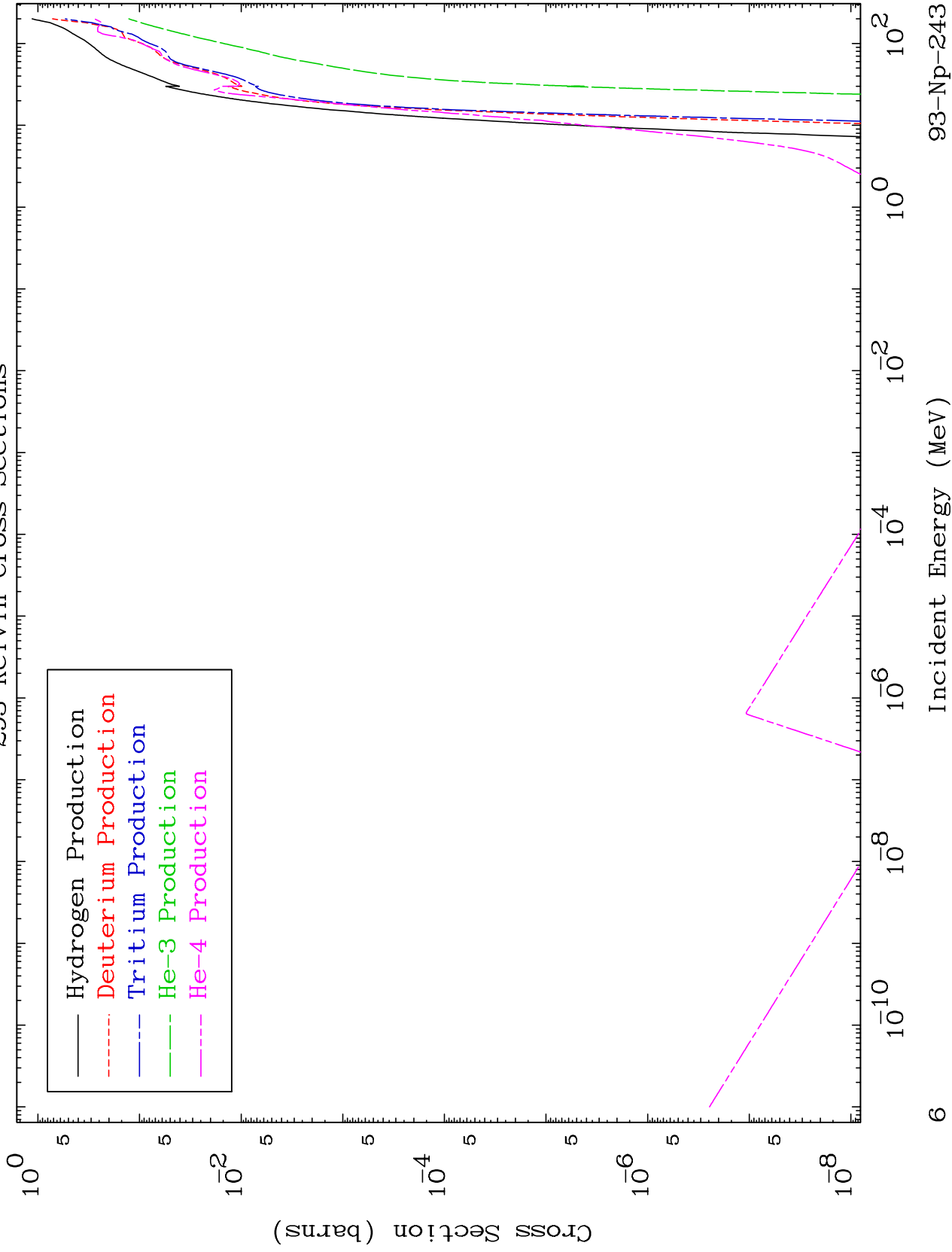
5

93-Np-243

MAT 9364

Particle Production
293 Kelvin Cross Sections

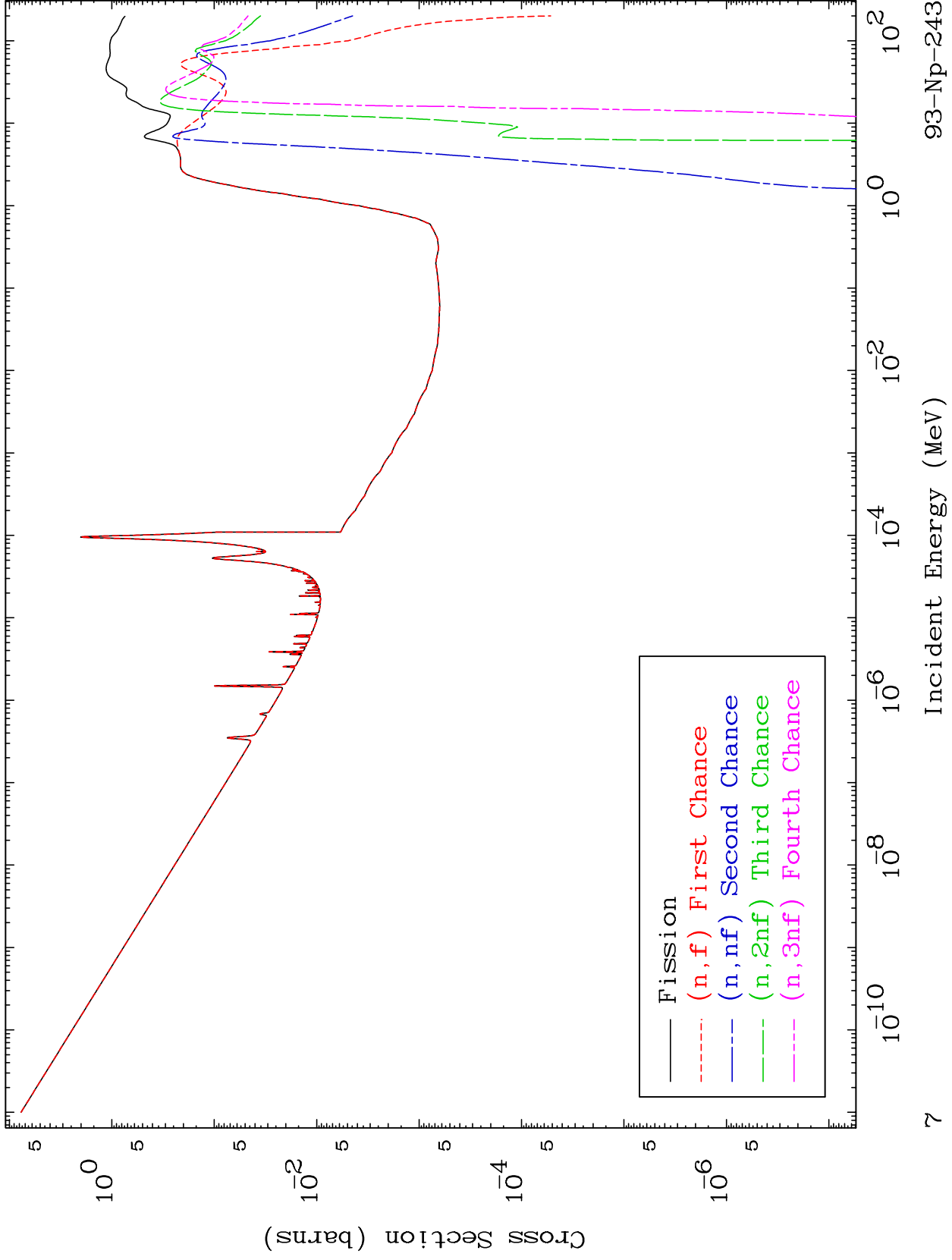
93-Np-243



MAT 9364

Fission
293 Kelvin Cross Sections

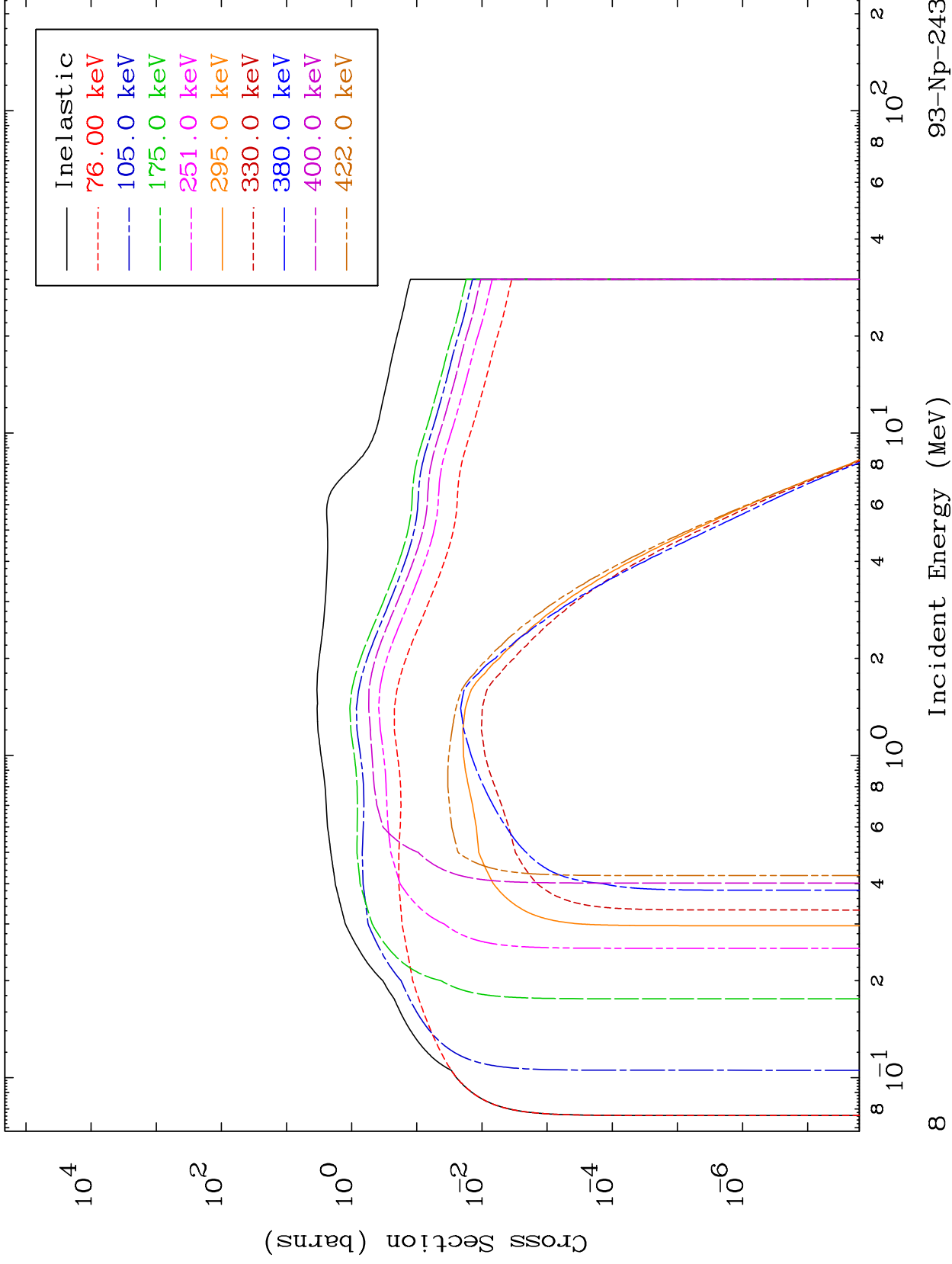
93-Np-243



MAT 9364

(n,n') Level
293 Kelvin Cross Sections

93-Np-243



93-Np-243

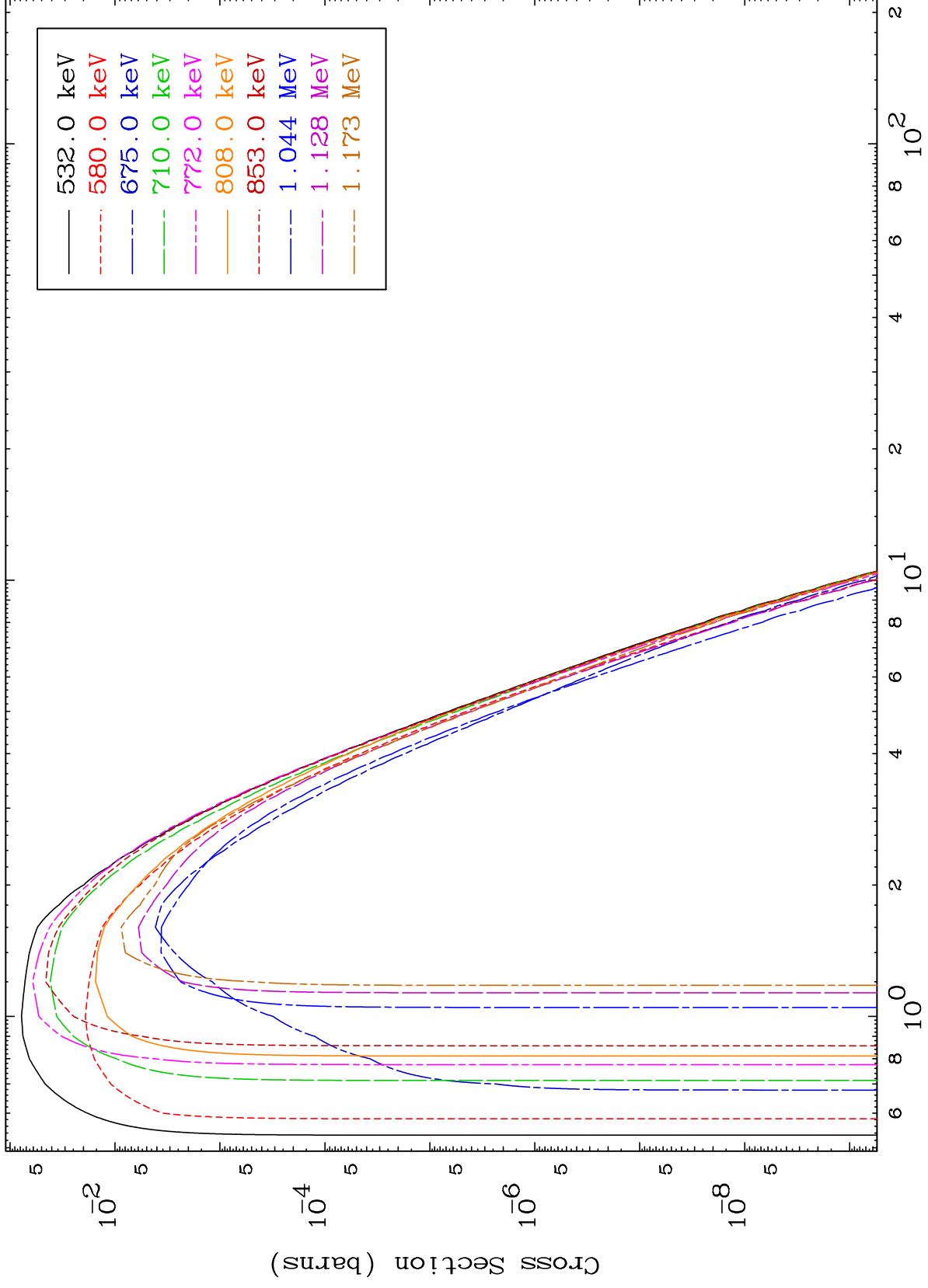
Incident Energy (MeV)

8

MAT 9364

(n,n') Level
293 Kelvin Cross Sections

93-Np-243



9

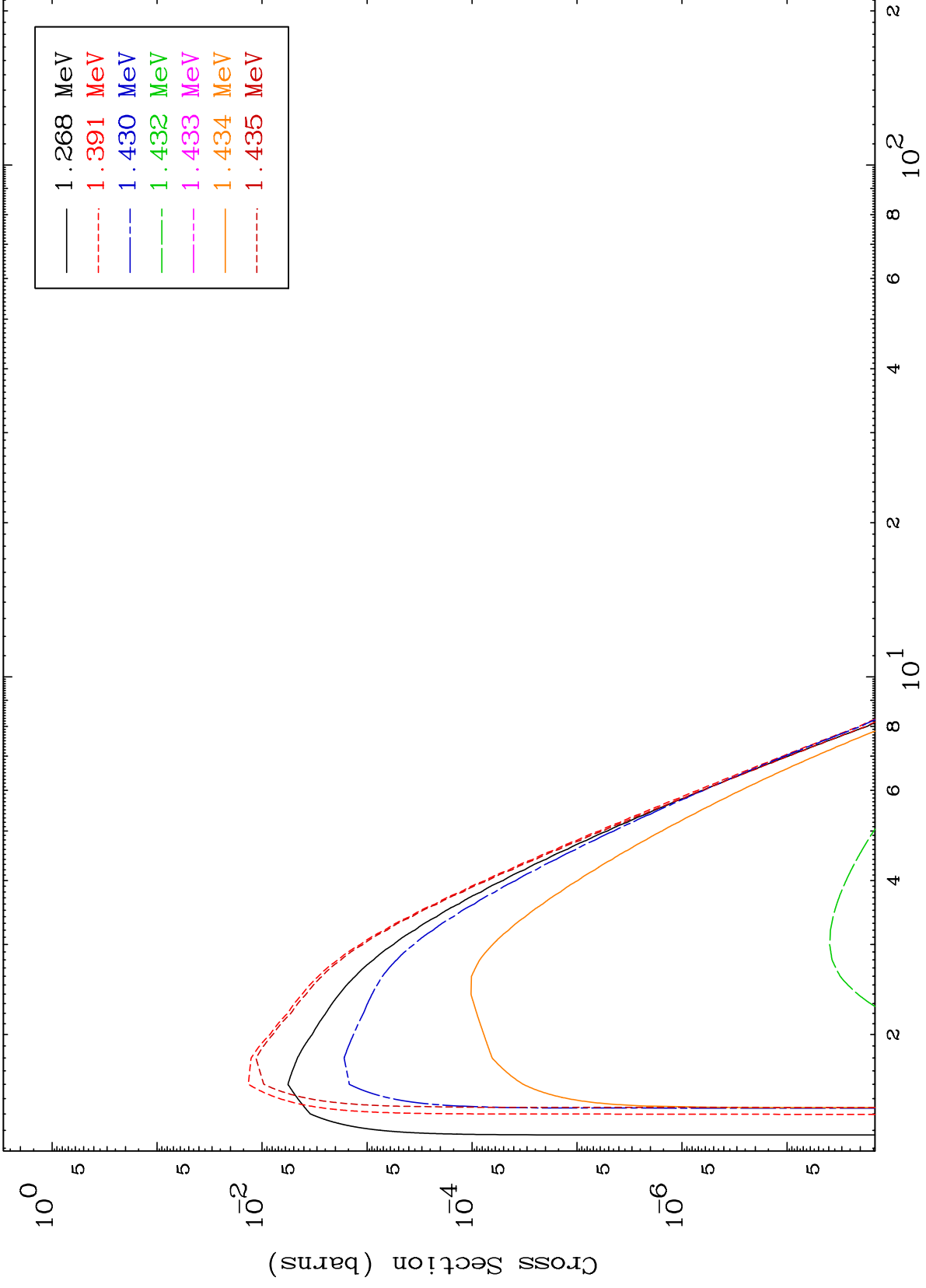
Incident Energy (MeV)

93-Np-243

MAT 9364

(n,n') Level
293 Kelvin Cross Sections

93-Np-243



10

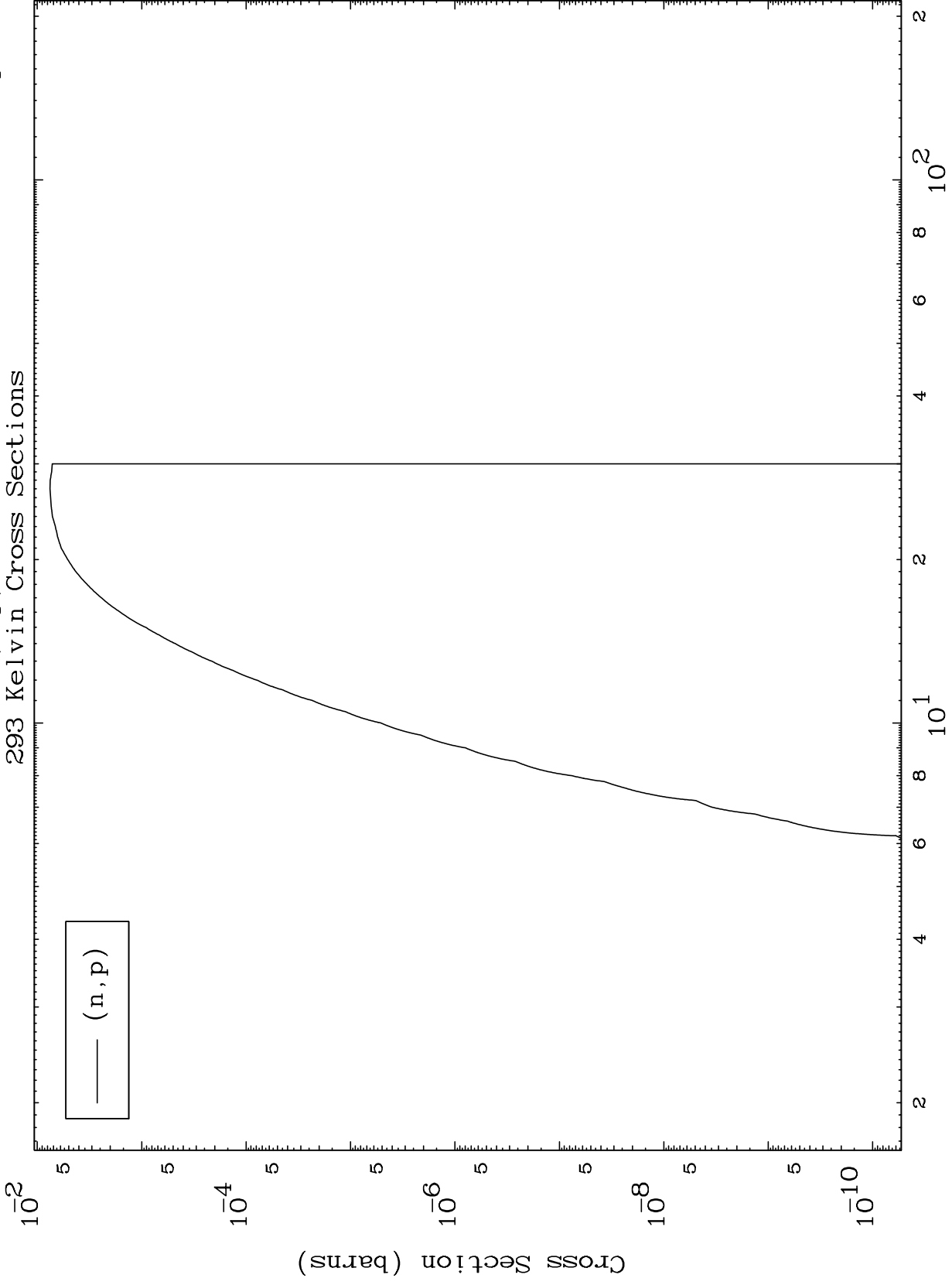
Incident Energy (MeV)

93-Np-243

MAT 9364

(n,p) Levels
293 Kelvin Cross Sections

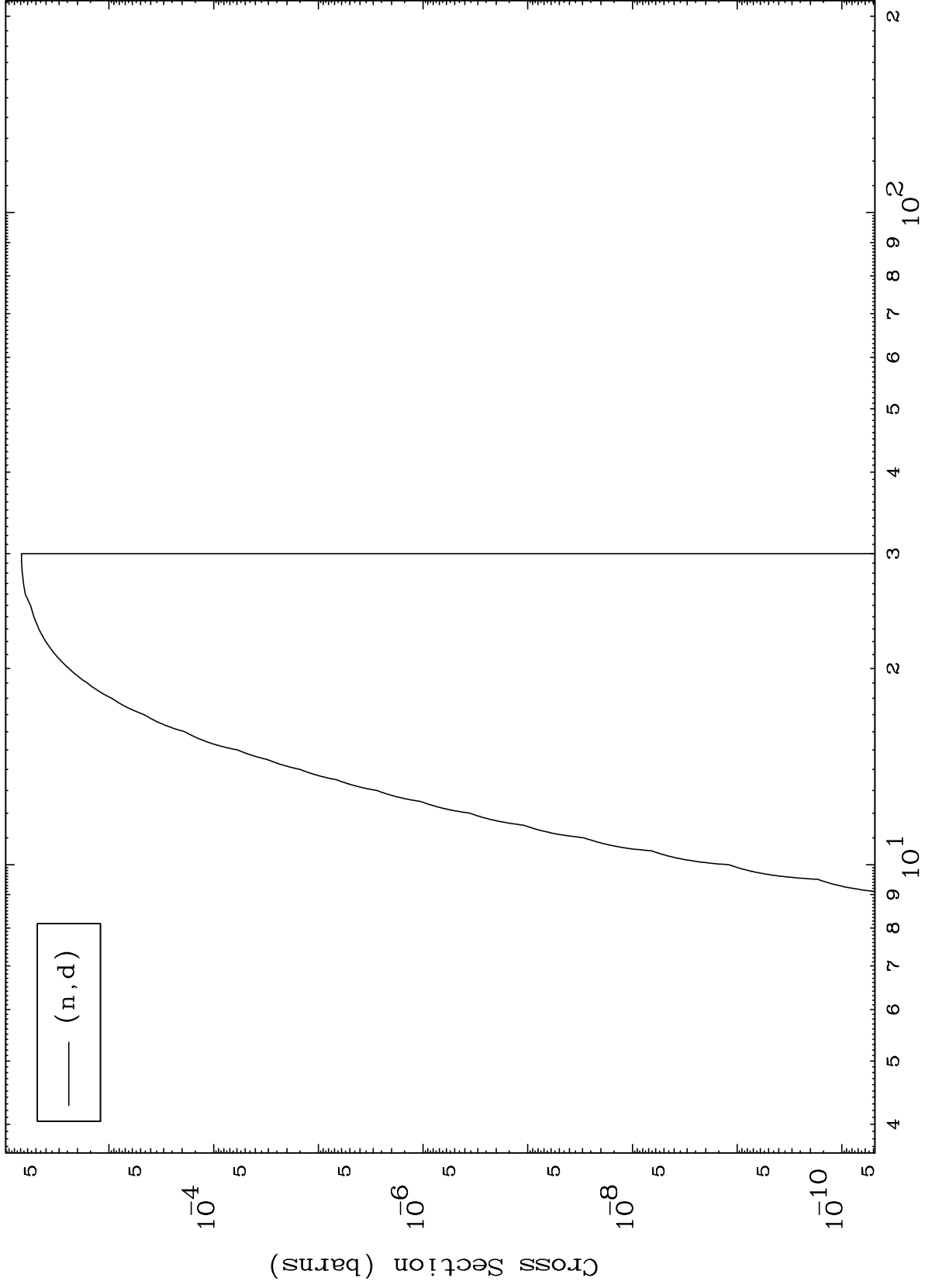
93-Np-243



MAT 9364

(n,d) Levels
293 Kelvin Cross Sections

93-Np-243



12

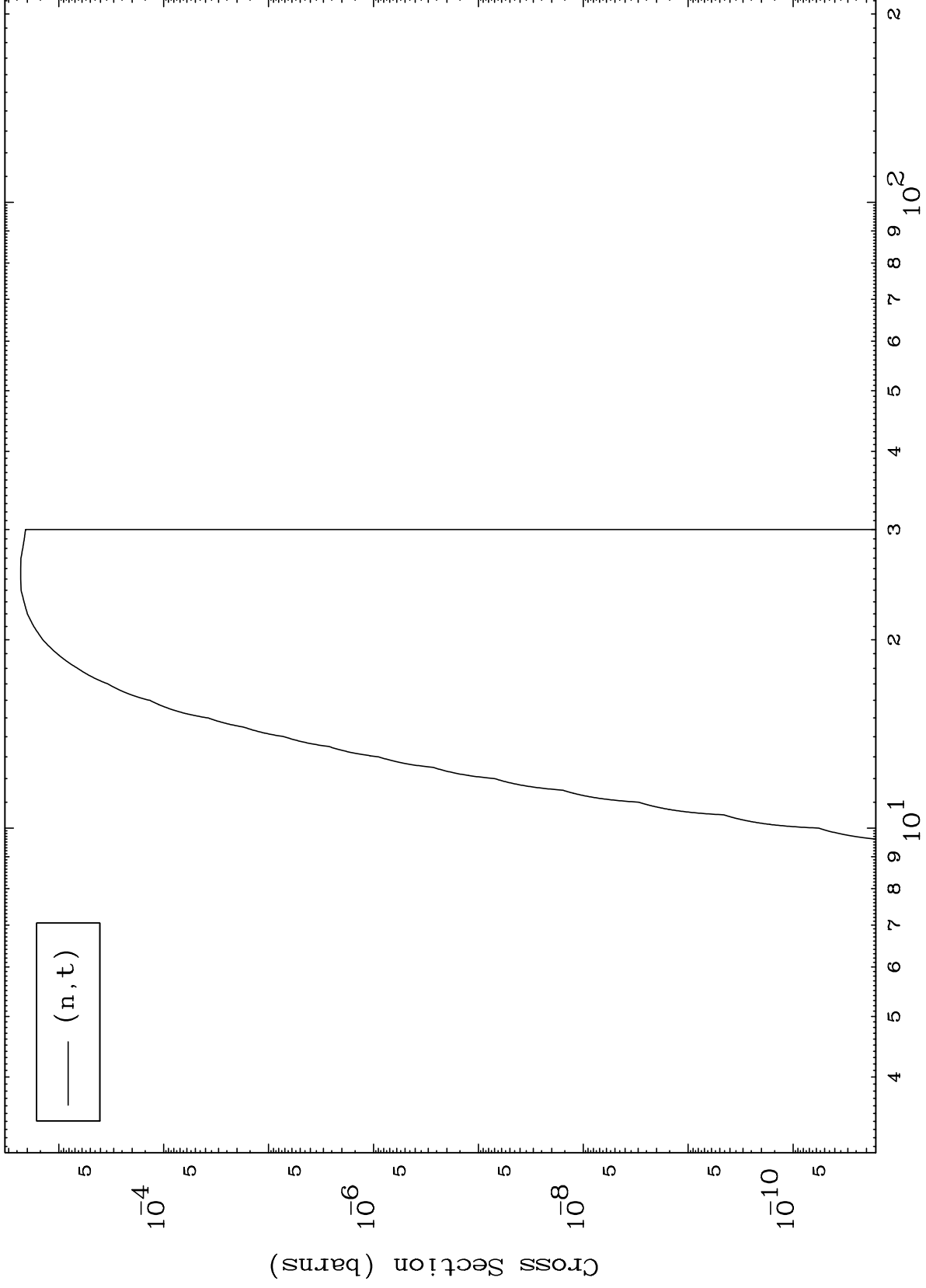
Incident Energy (MeV)

93-Np-243

MAT 9364

(n,t) Levels
293 Kelvin Cross Sections

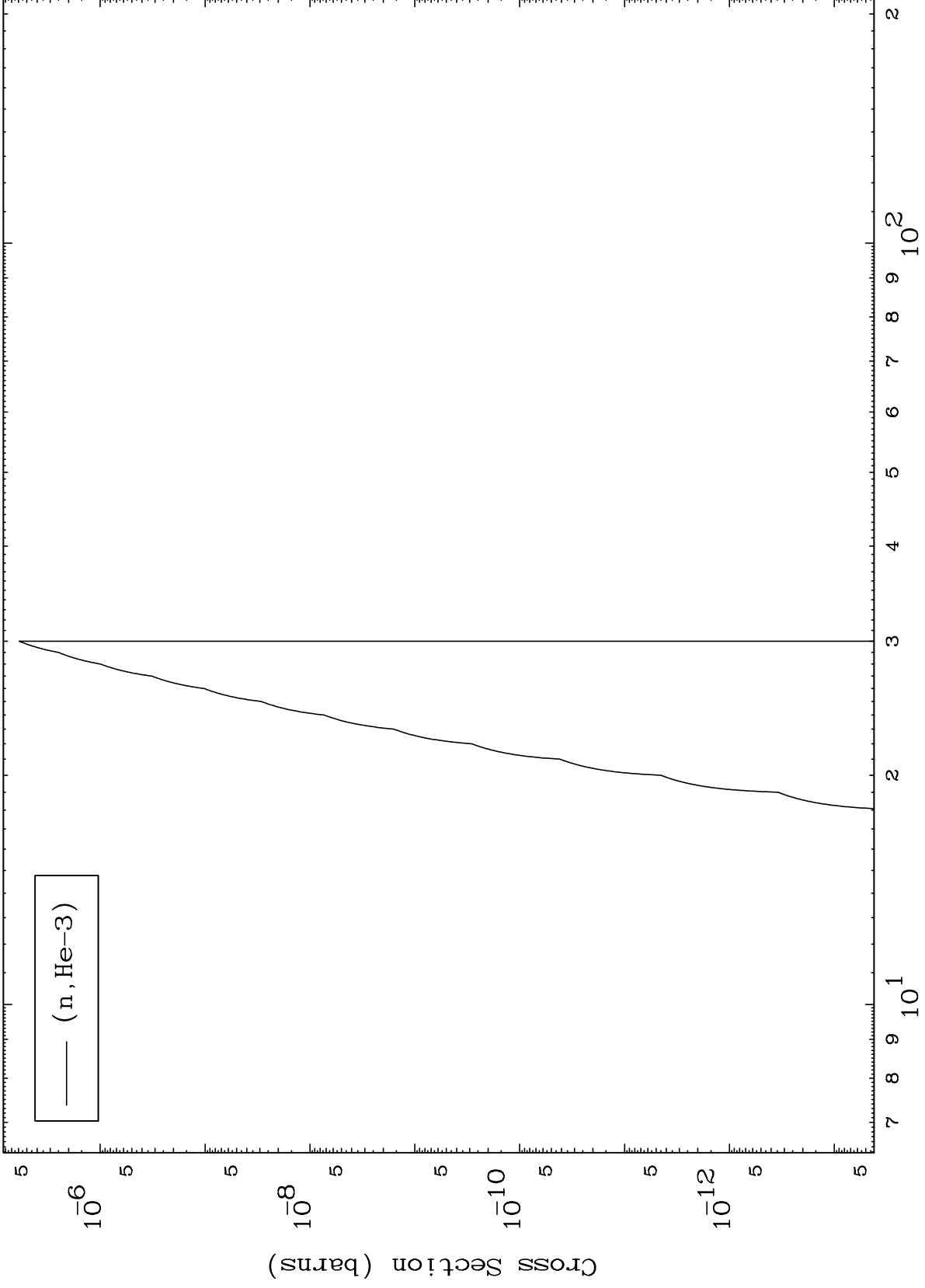
93-Np-243



MAT 9364

(n,He3) Levels
293 Kelvin Cross Sections

93-Np-243



14

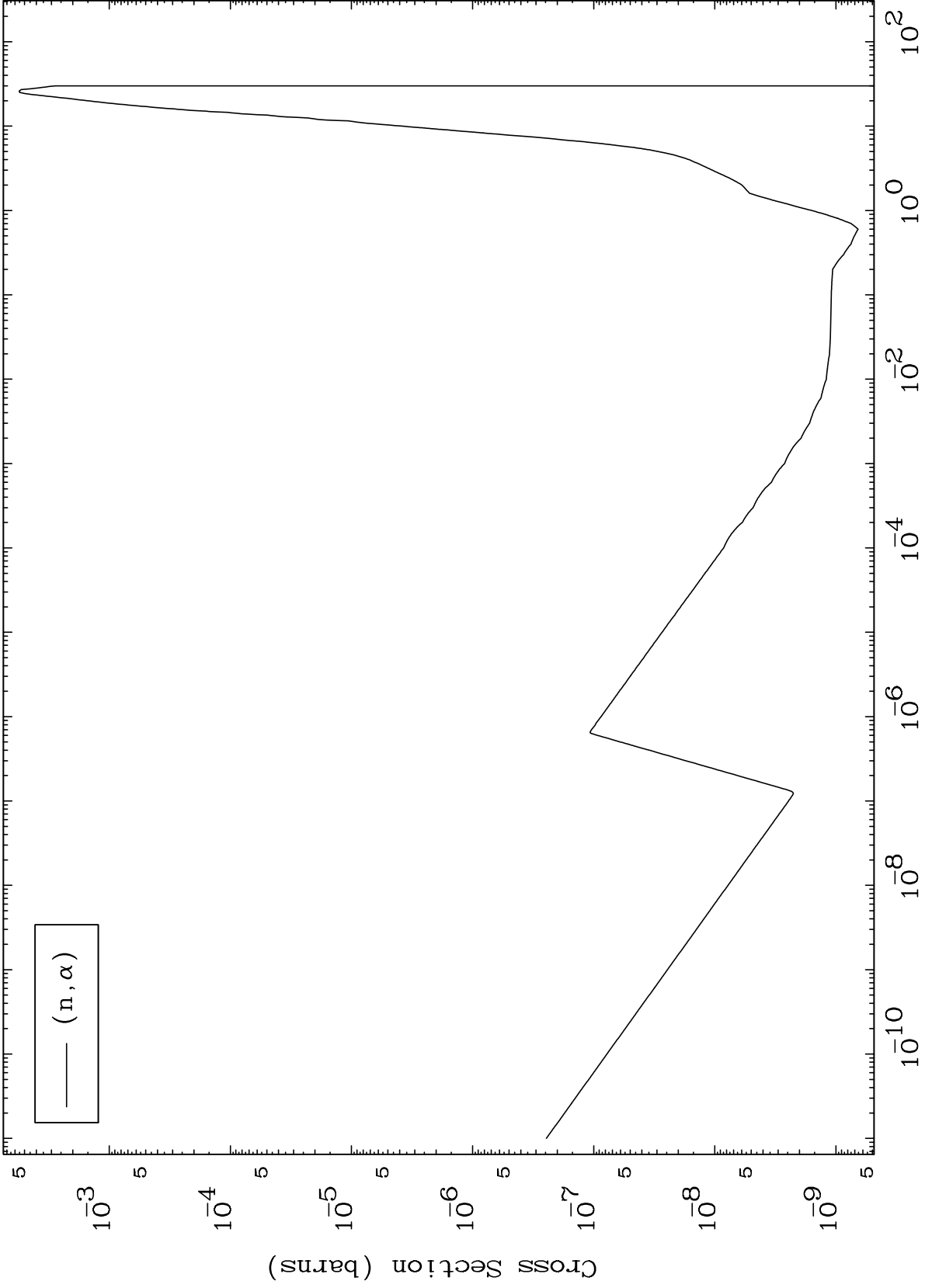
Incident Energy (MeV)

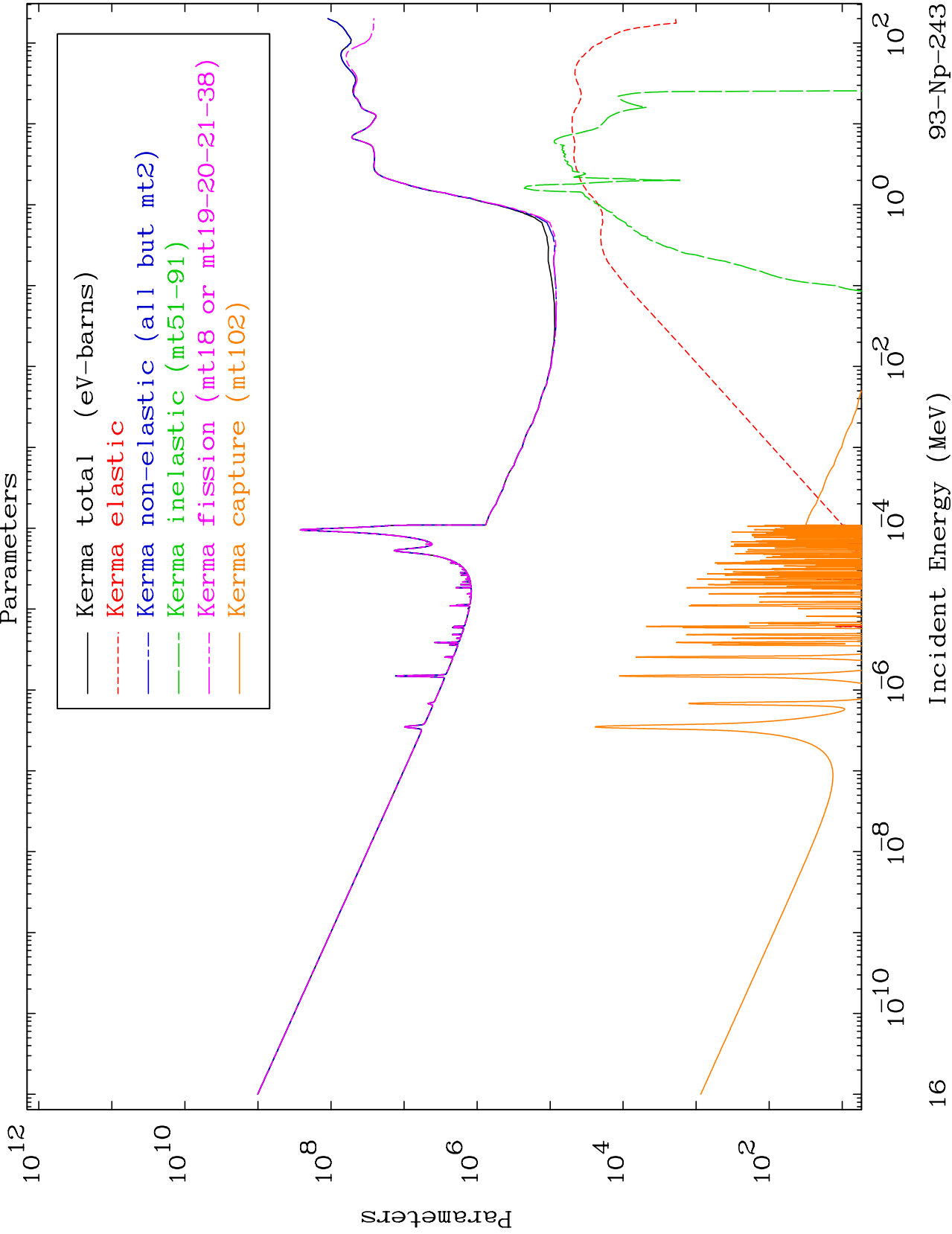
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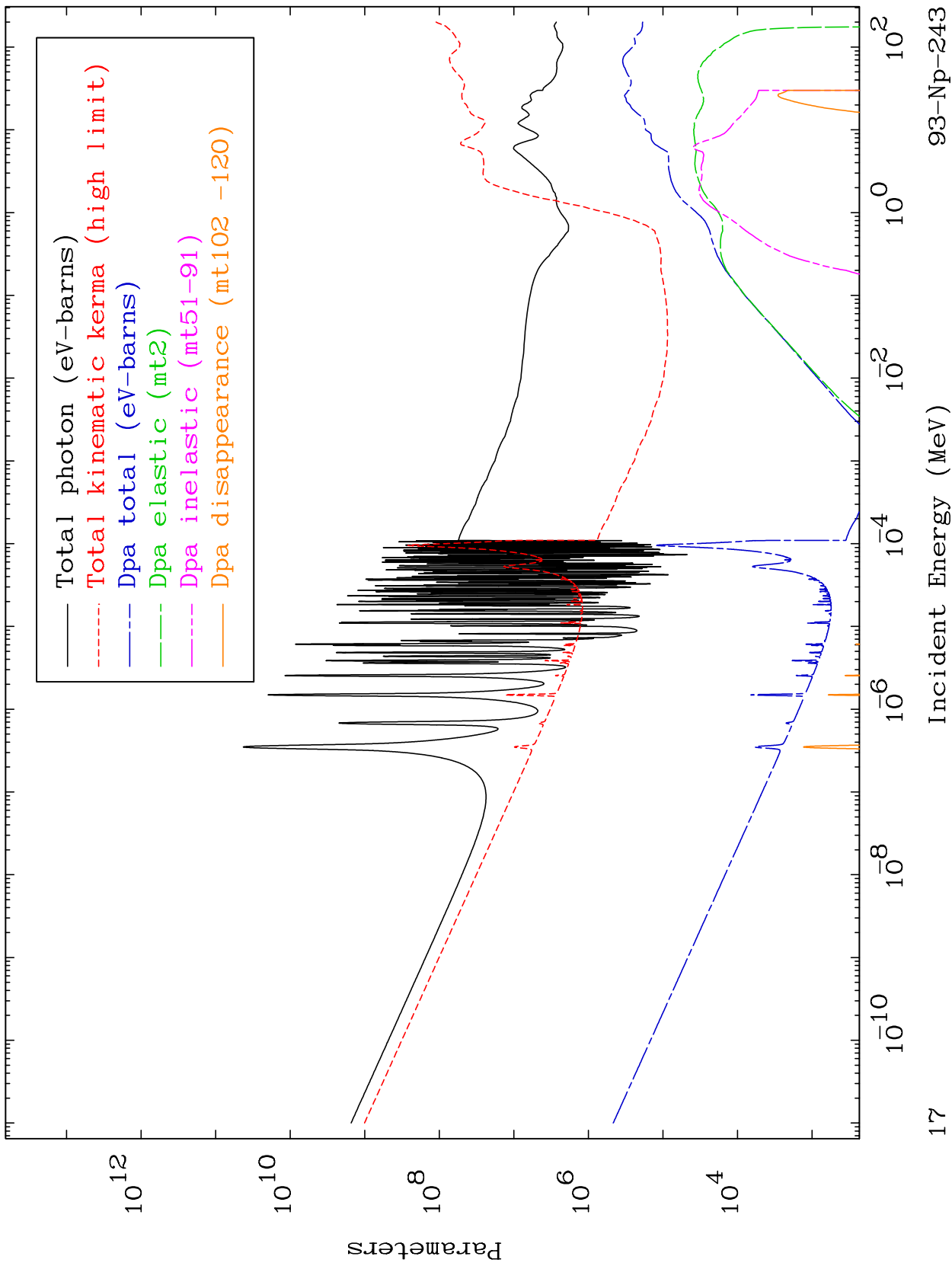
MAT 9364

(n, α) Levels
293 Kelvin Cross Sections

93-Np-243



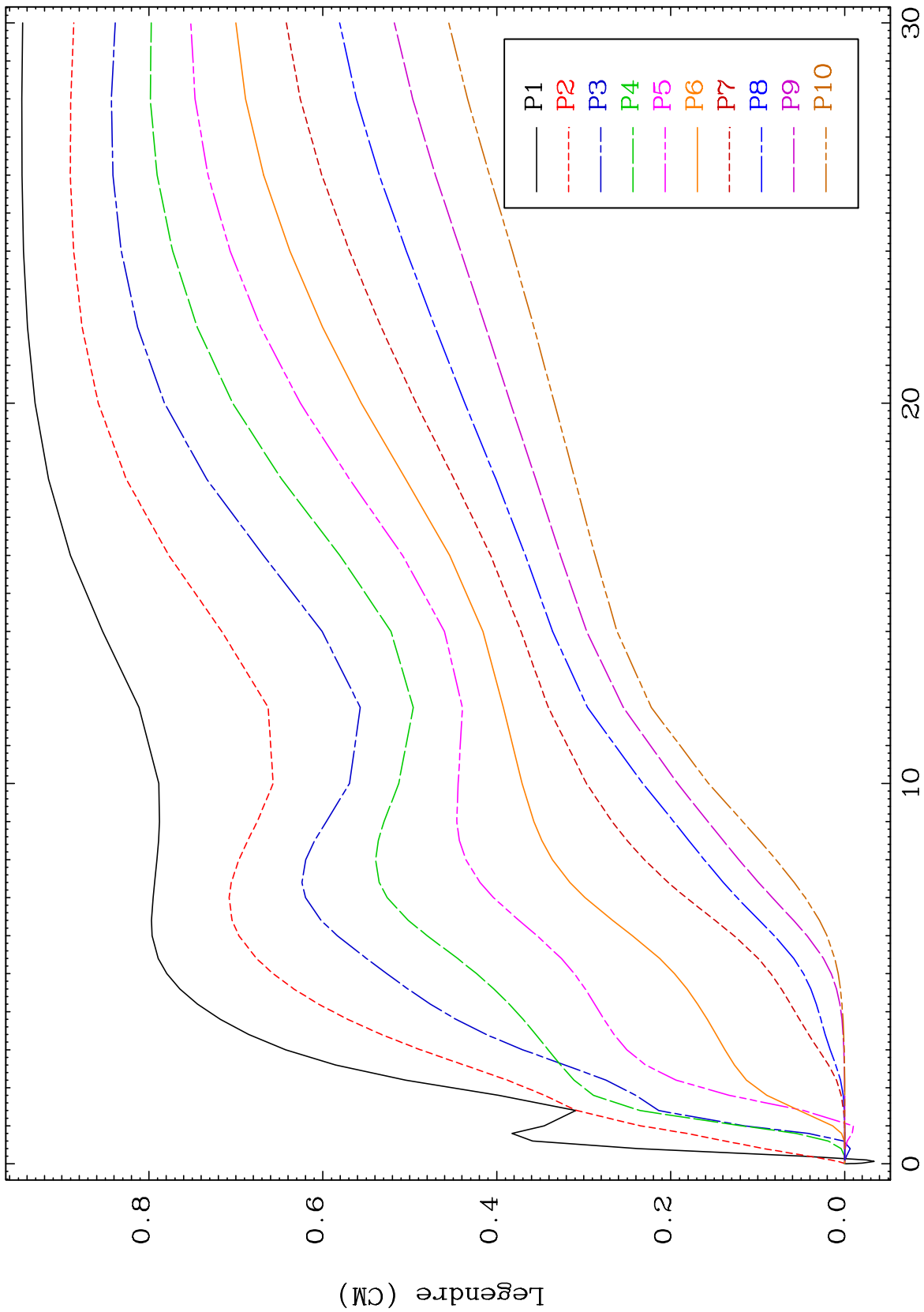




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93-Np-243

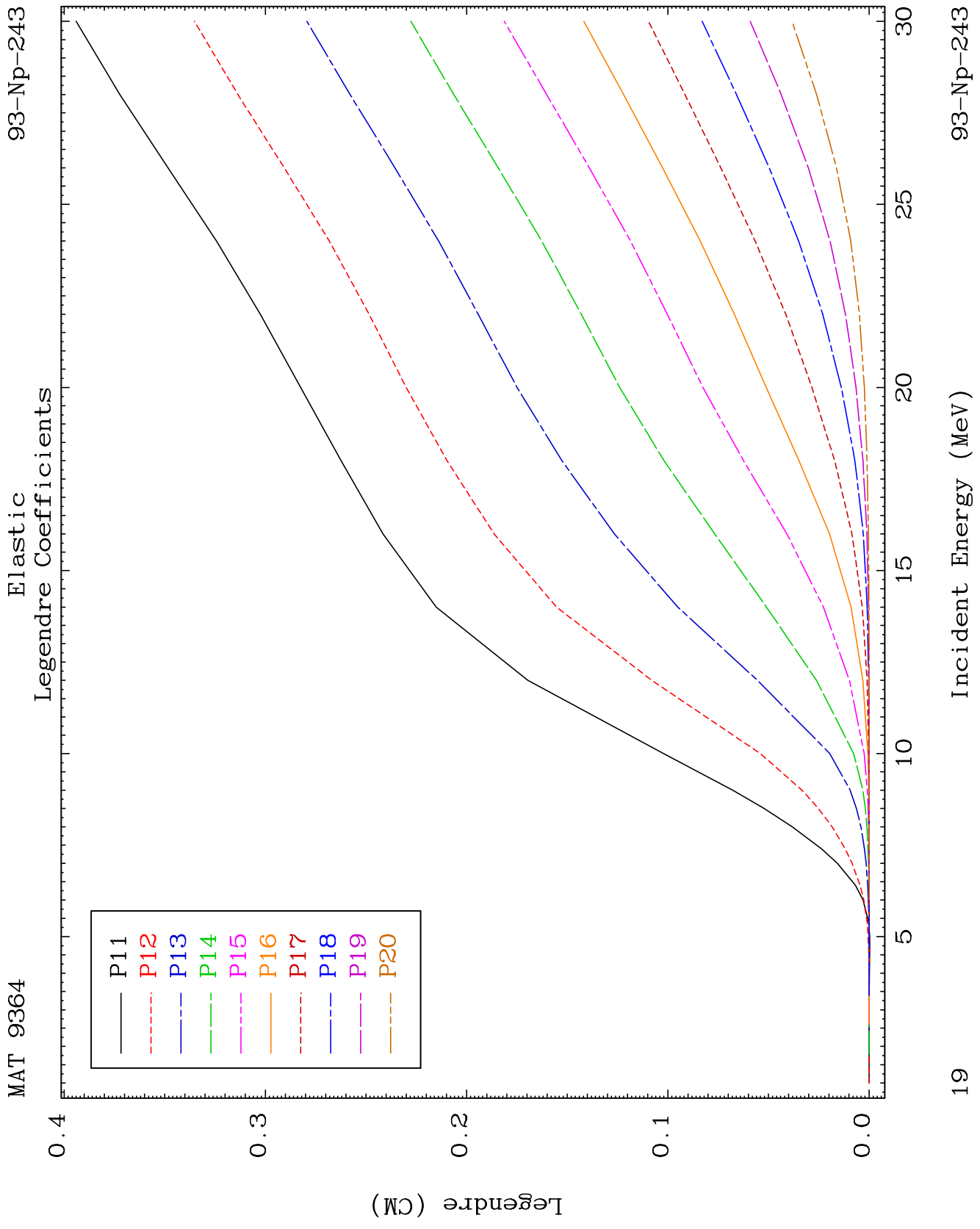
Elastic Legendre Coefficients



93-Np-243

Incident Energy (MeV)

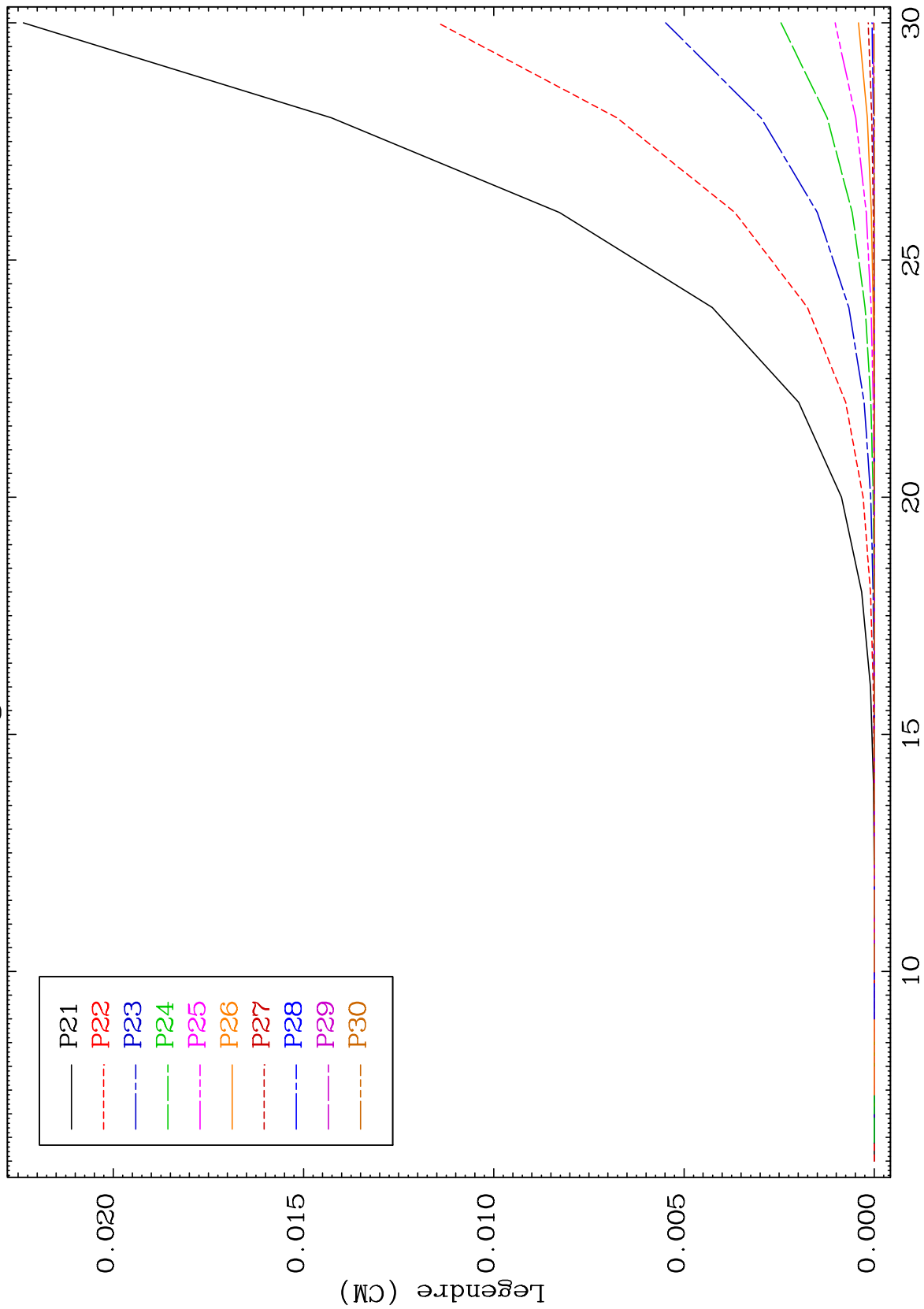
18



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Elastic Legendre Coefficients

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20

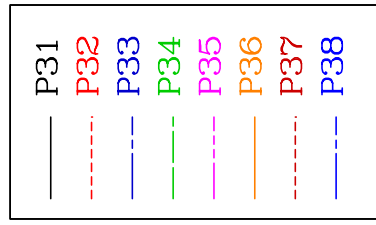
Incident Energy (MeV)

93-Np-243

MAT 9364

Elastic Legendre Coefficients

93-Np-243



$\times 10^{-6}$

2.0

1.5

1.0

0.5

0.0

Legendre (CM)

15

20

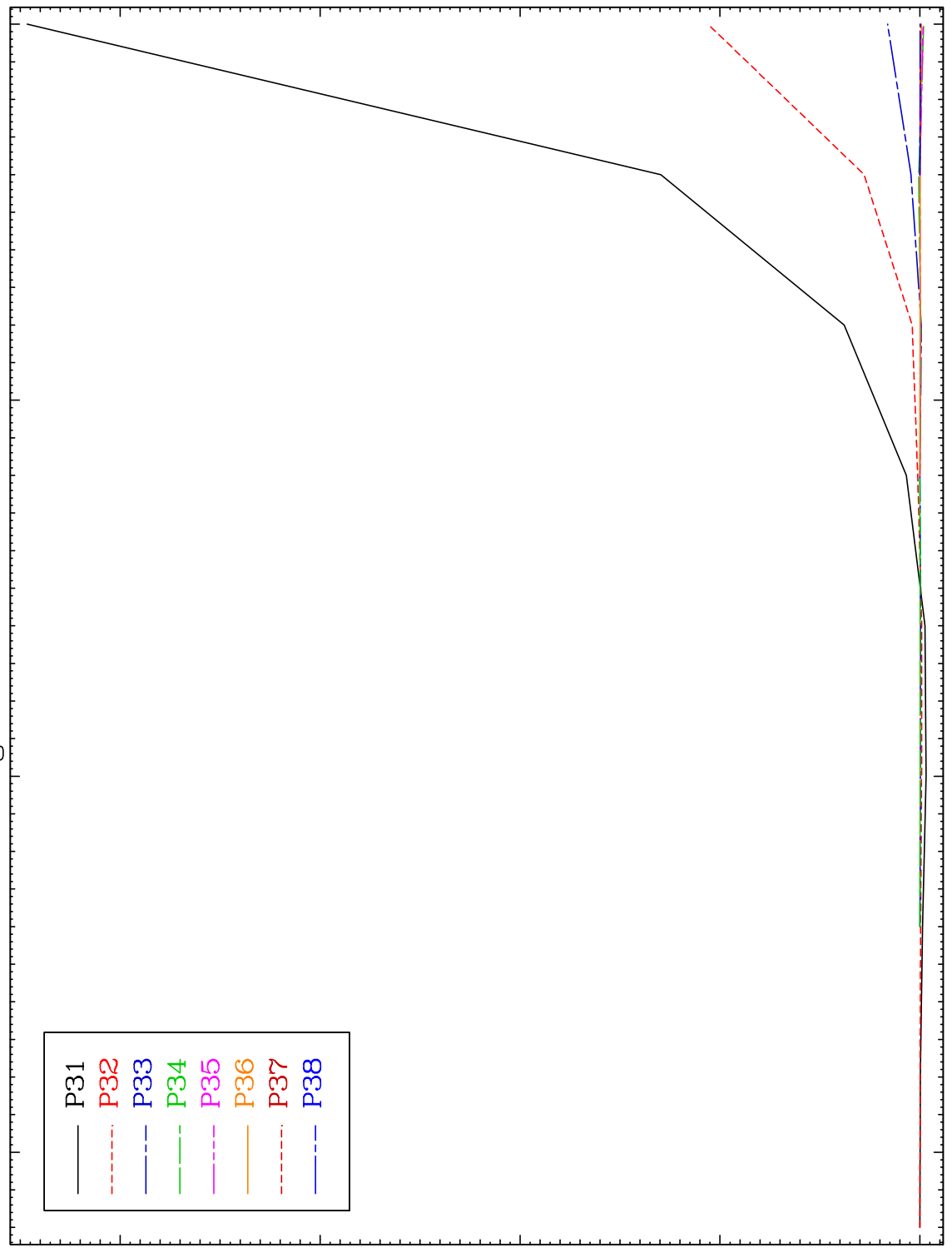
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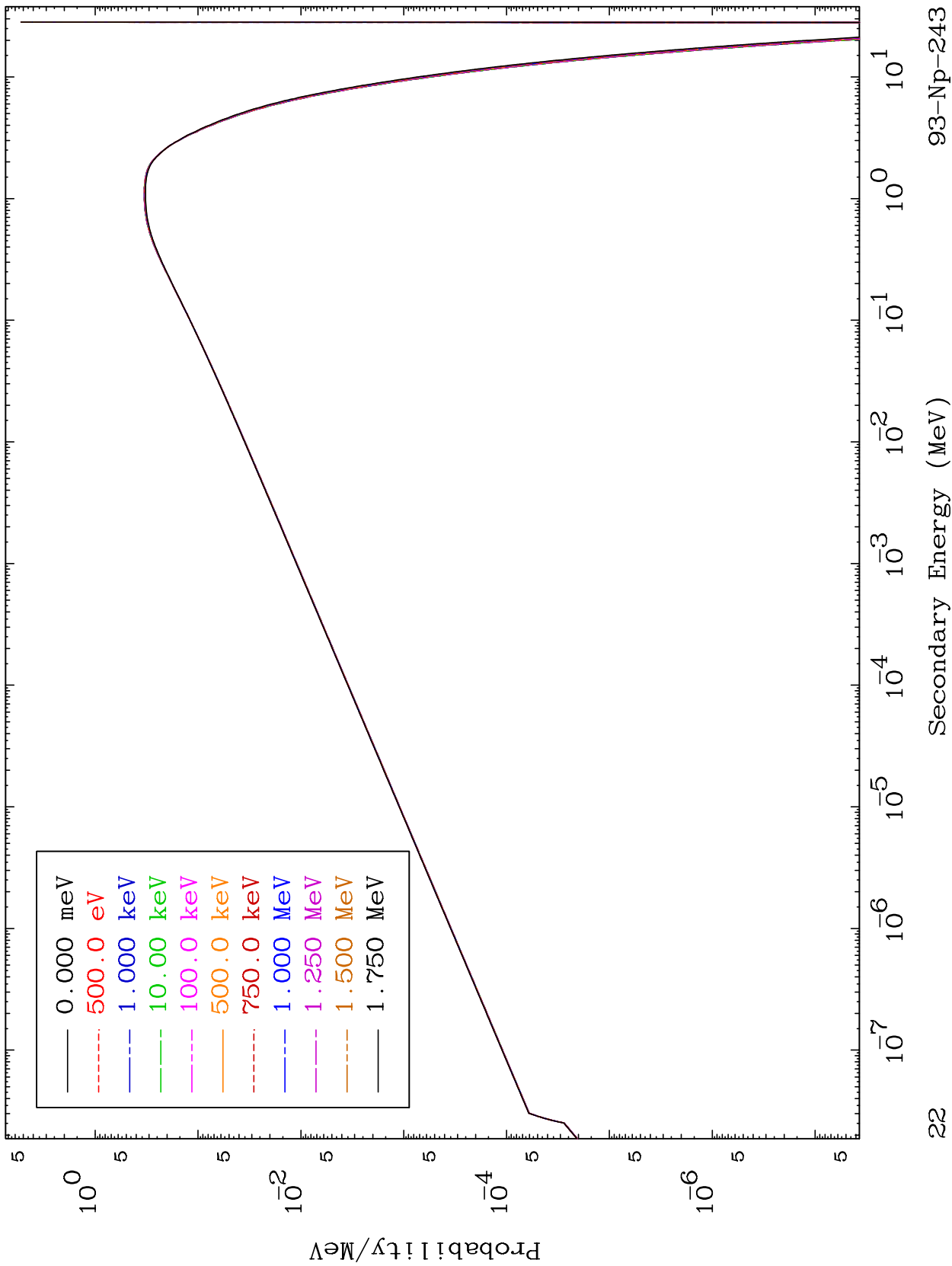
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21

Incident Energy (MeV)

93-Np-243

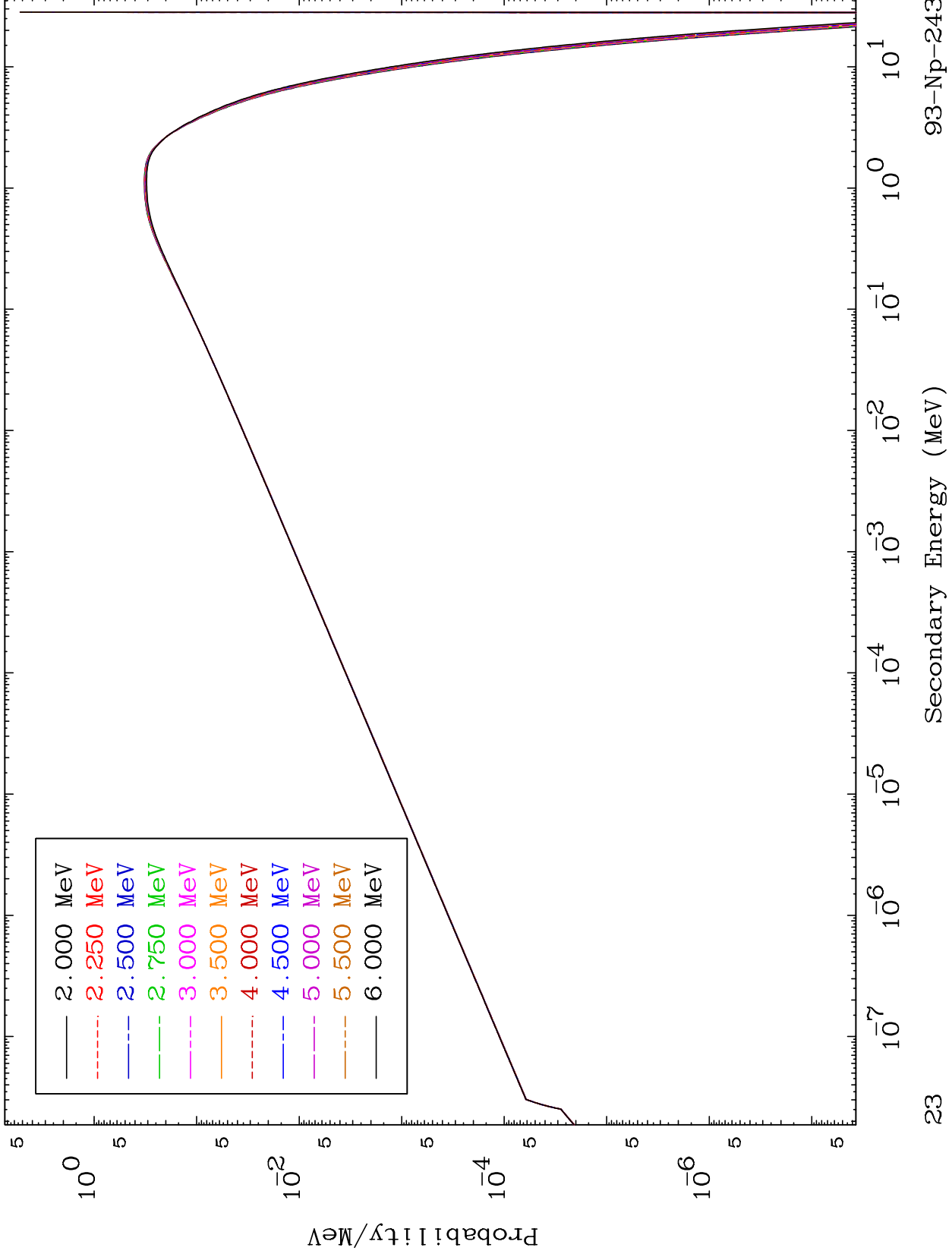




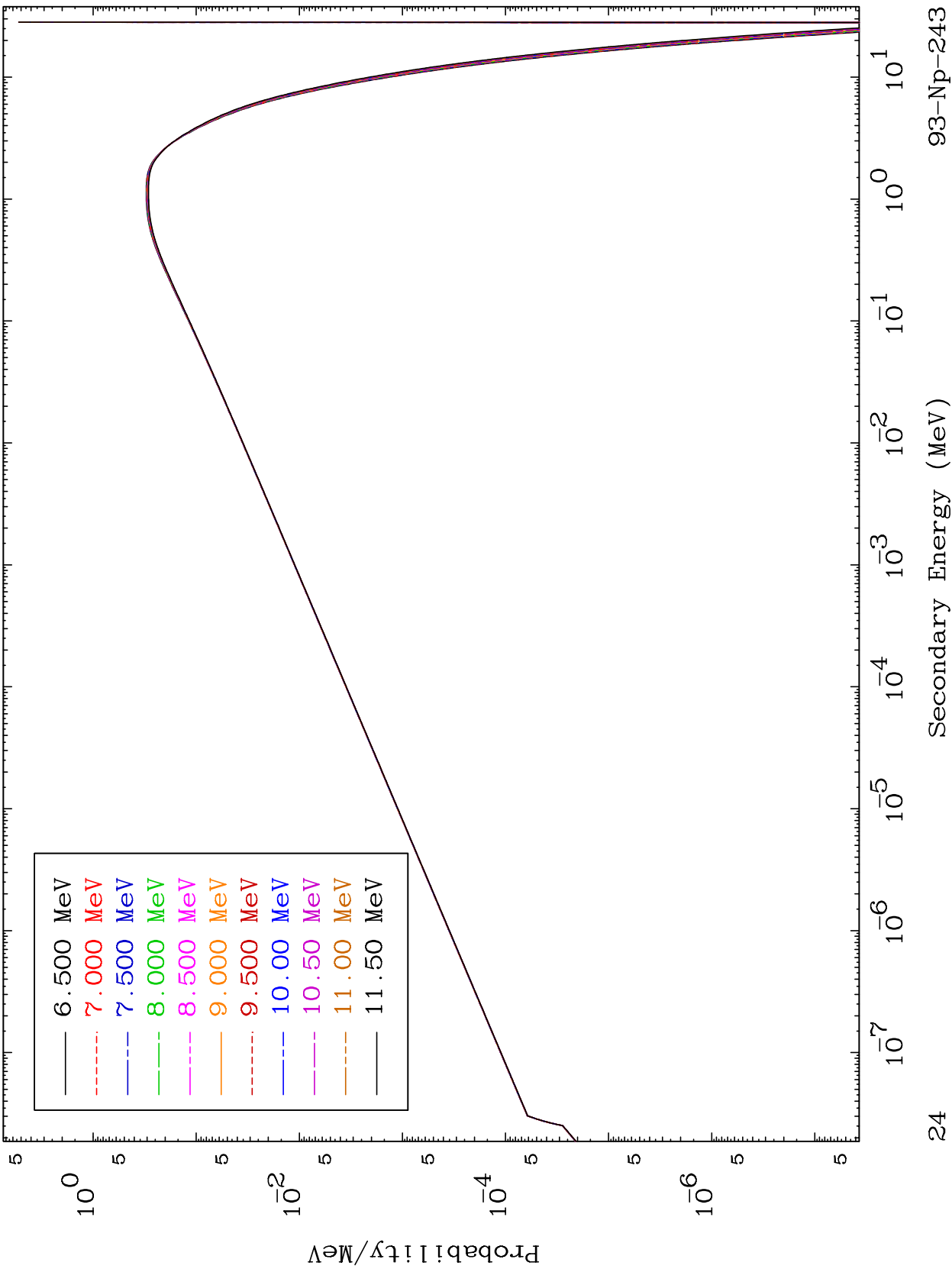
MAT 9364

Fission Energy Distribution

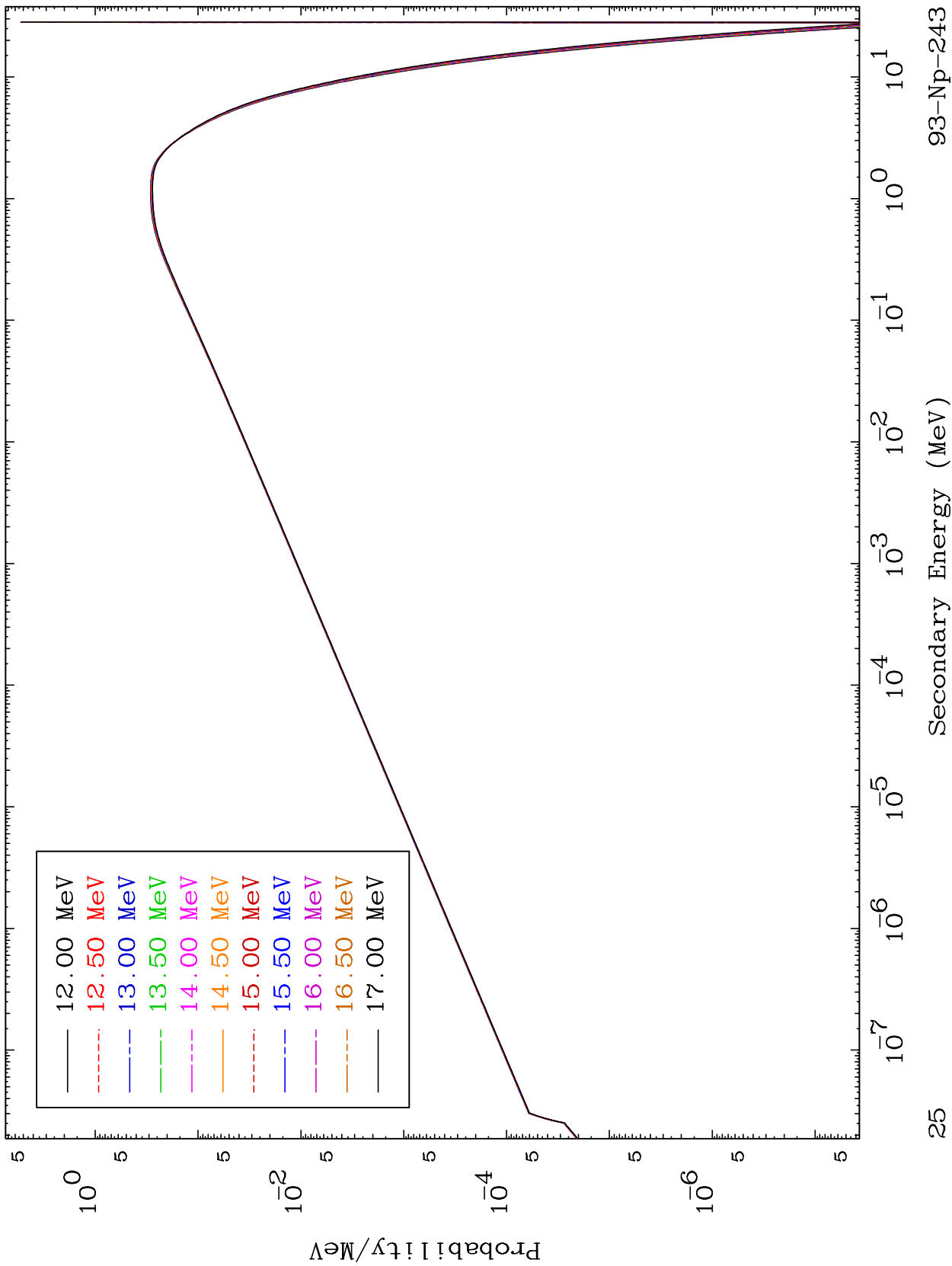
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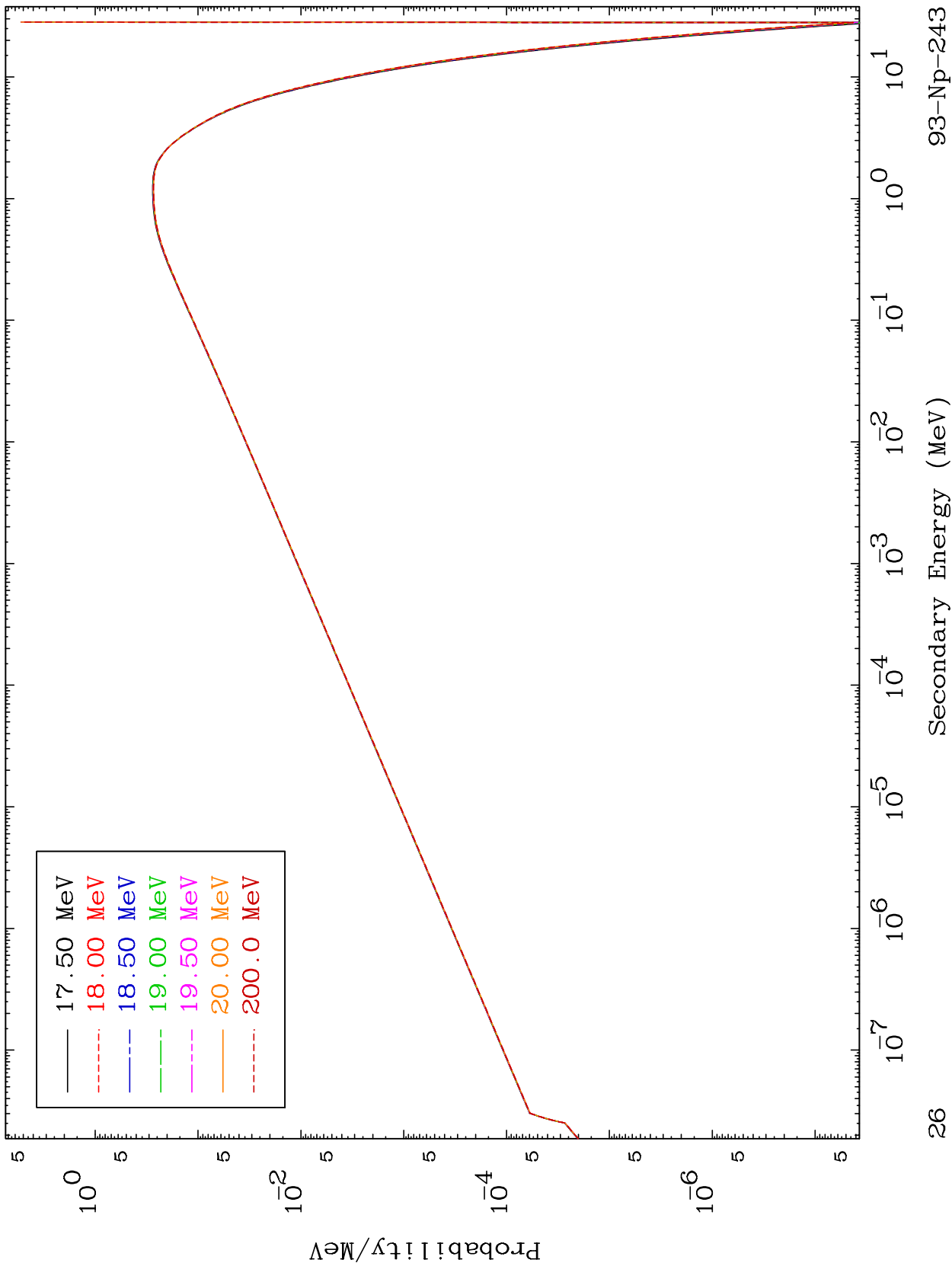
Fission
Energy Distribution



Fission
Energy Distribution



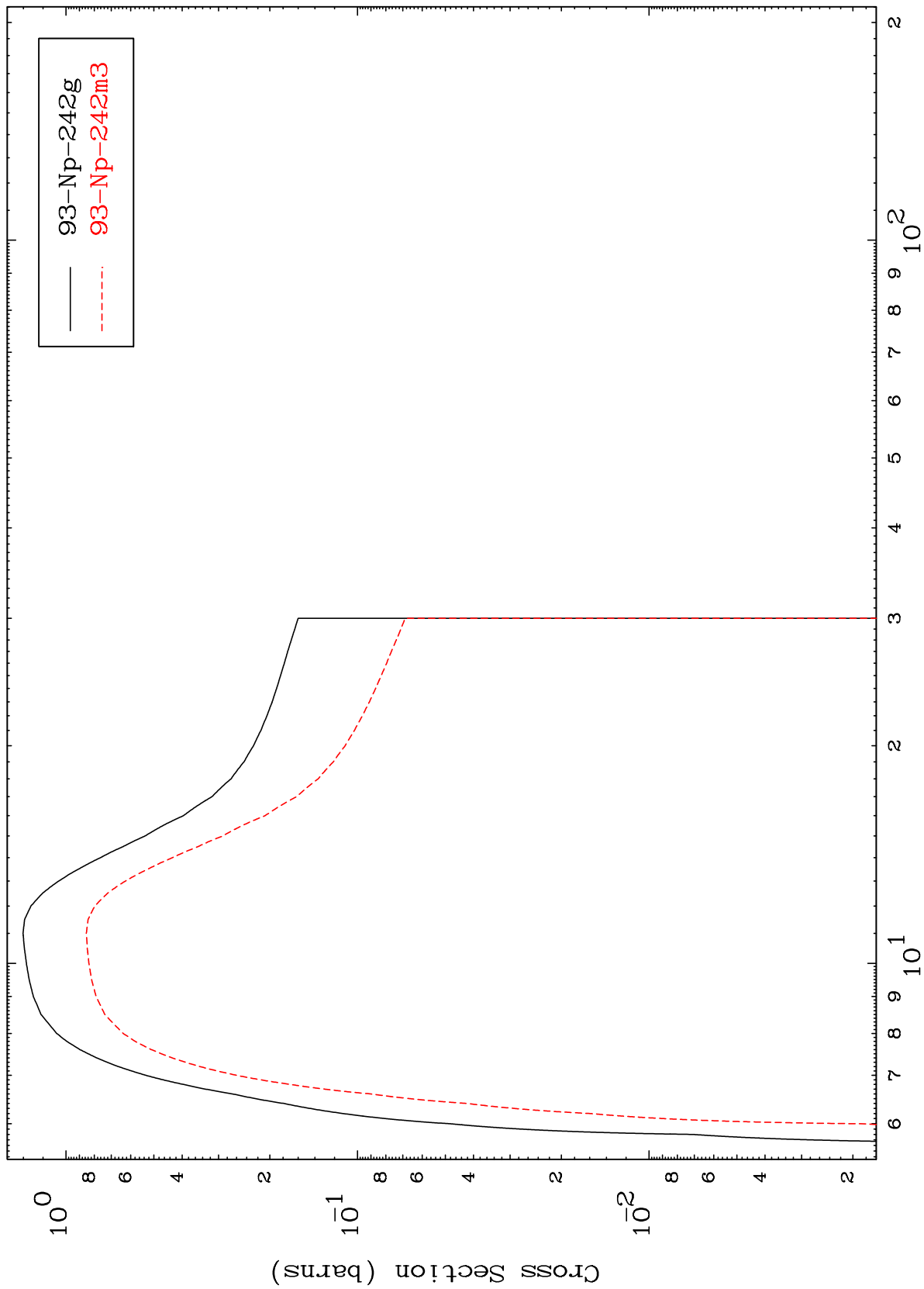
Fission
Energy Distribution



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93-Np-243

(n,2n)
Radionuclide Production Cross Section



27

Incident Energy (MeV)

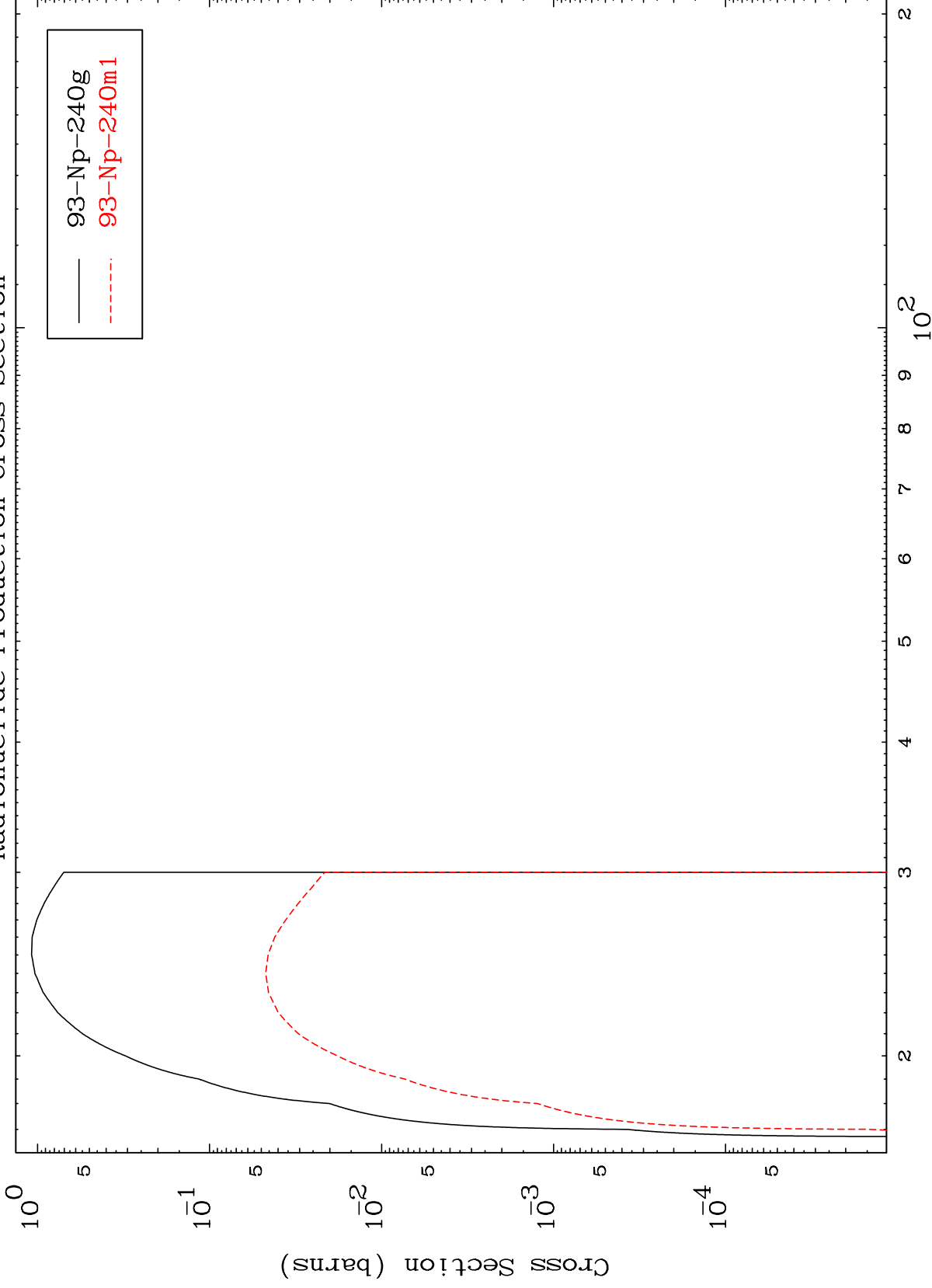
93-Np-243

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(n,4n)

93-Np-243

Radionuclide Production Cross Section



28

Incident Energy (MeV)

93-Np-243