

Program EVALPLOT
(Version 2015-2)

by

Dermott E. Cullen
(Present Contact Information)

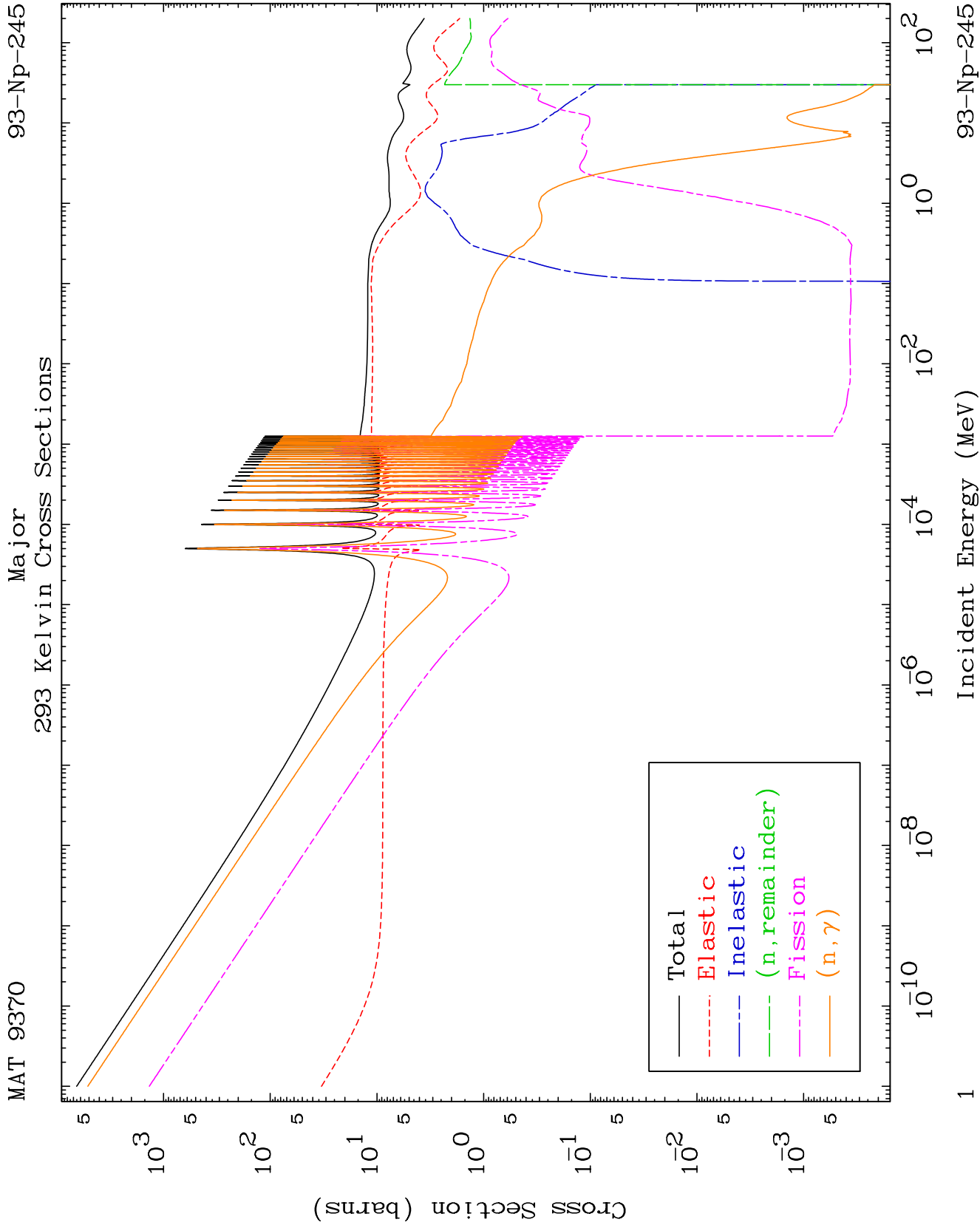
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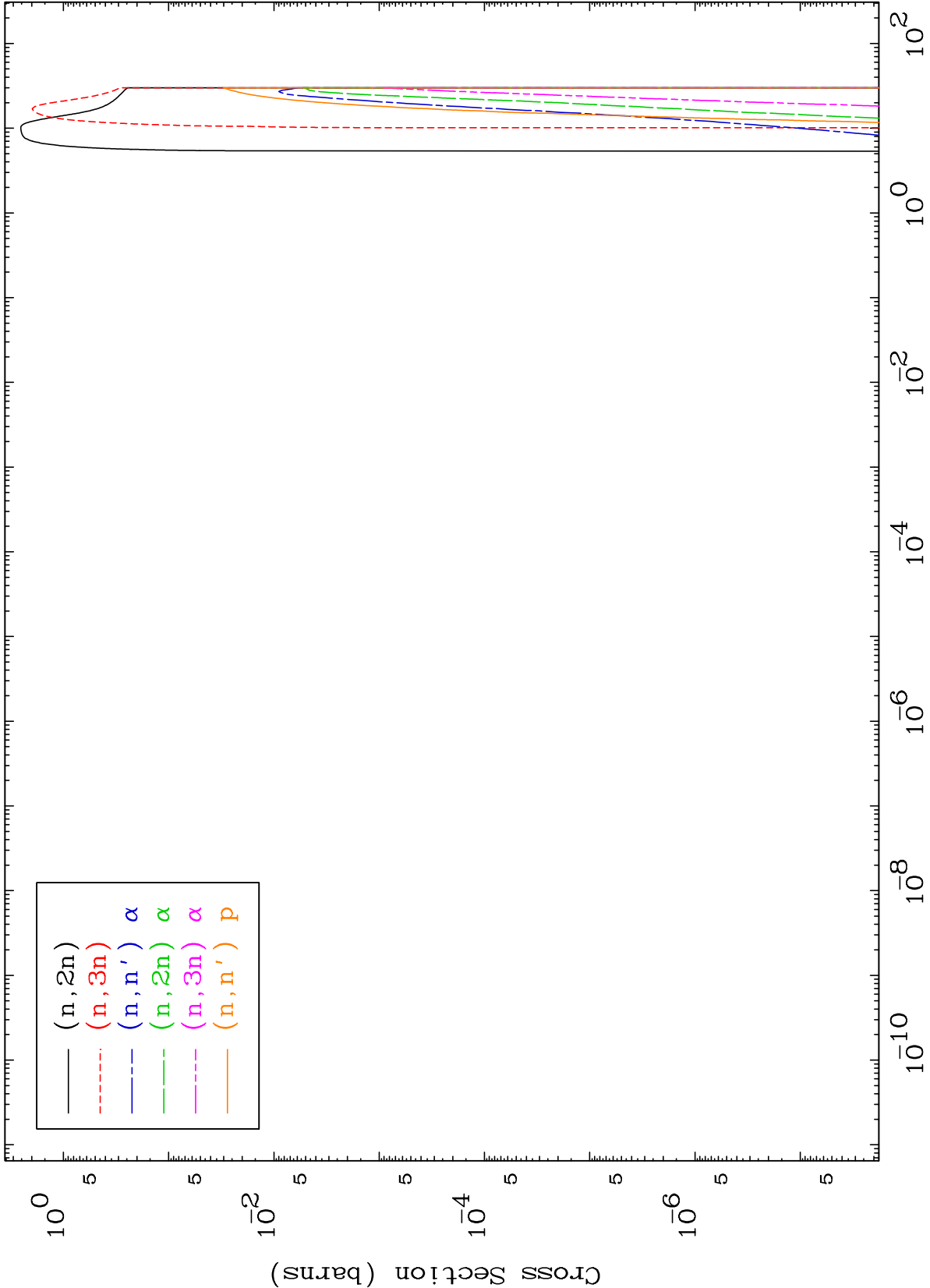
Press Mouse Button to Start

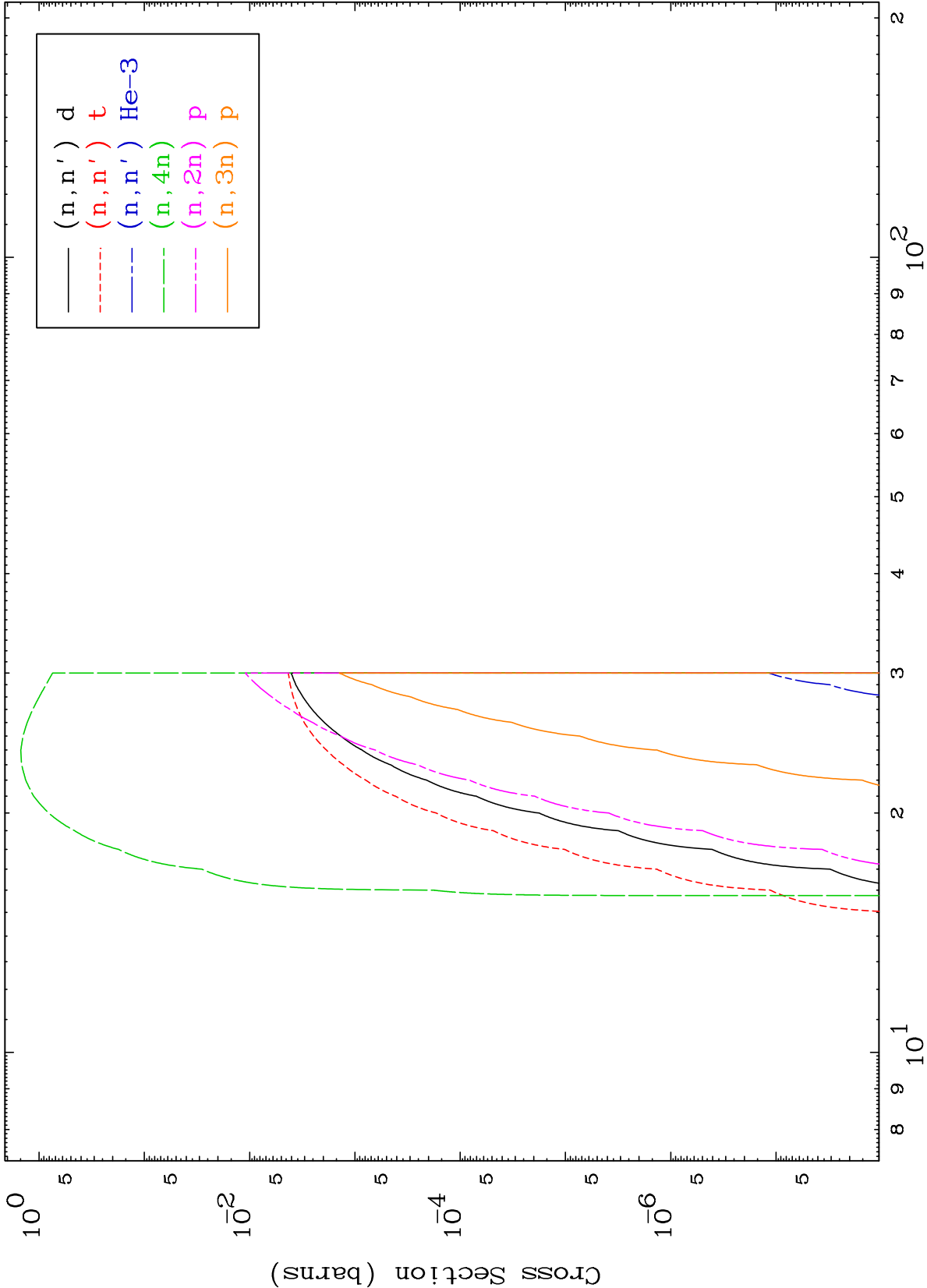


MAT 9370

Neutron Production
293 Kelvin Cross Sections

93-Np-245

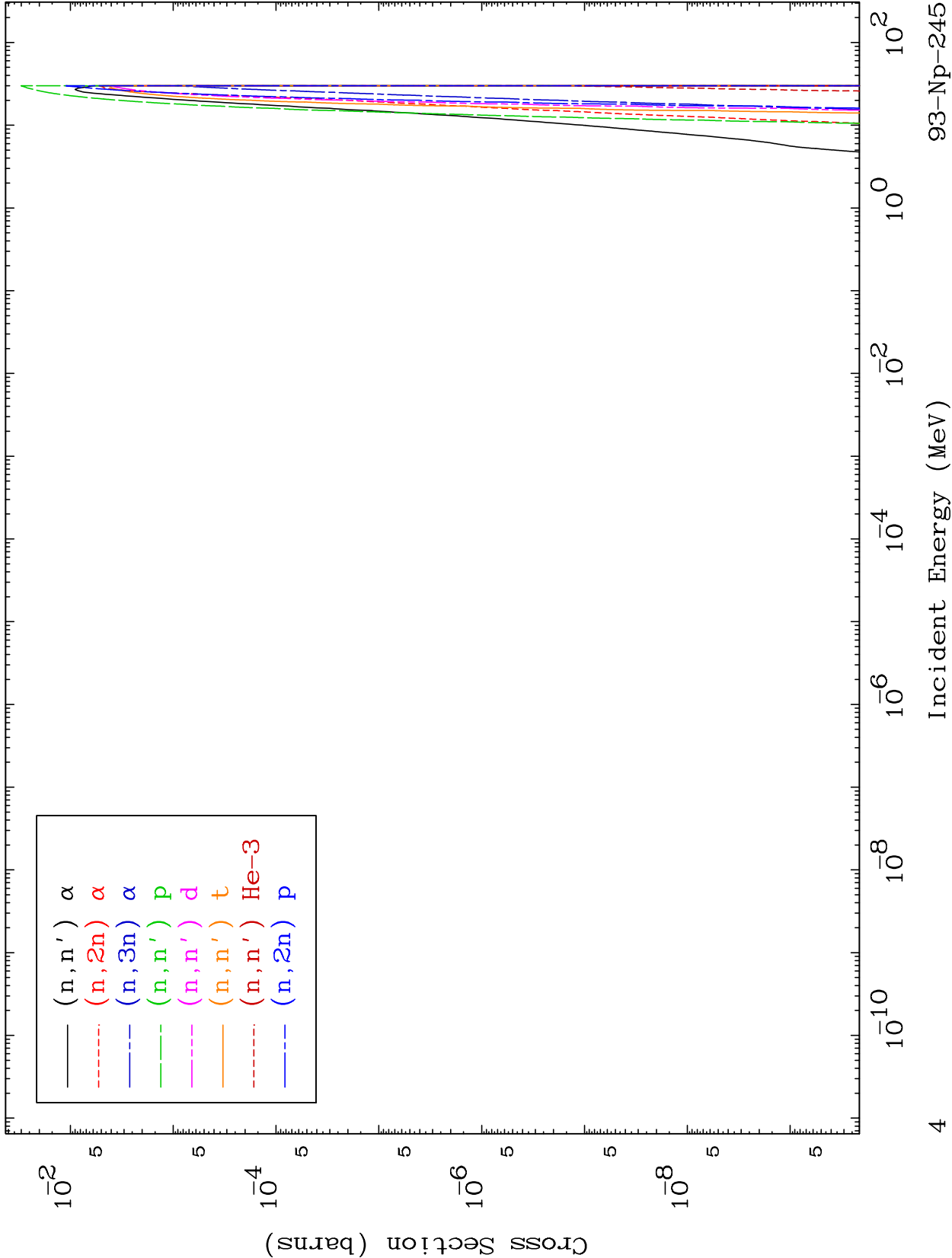


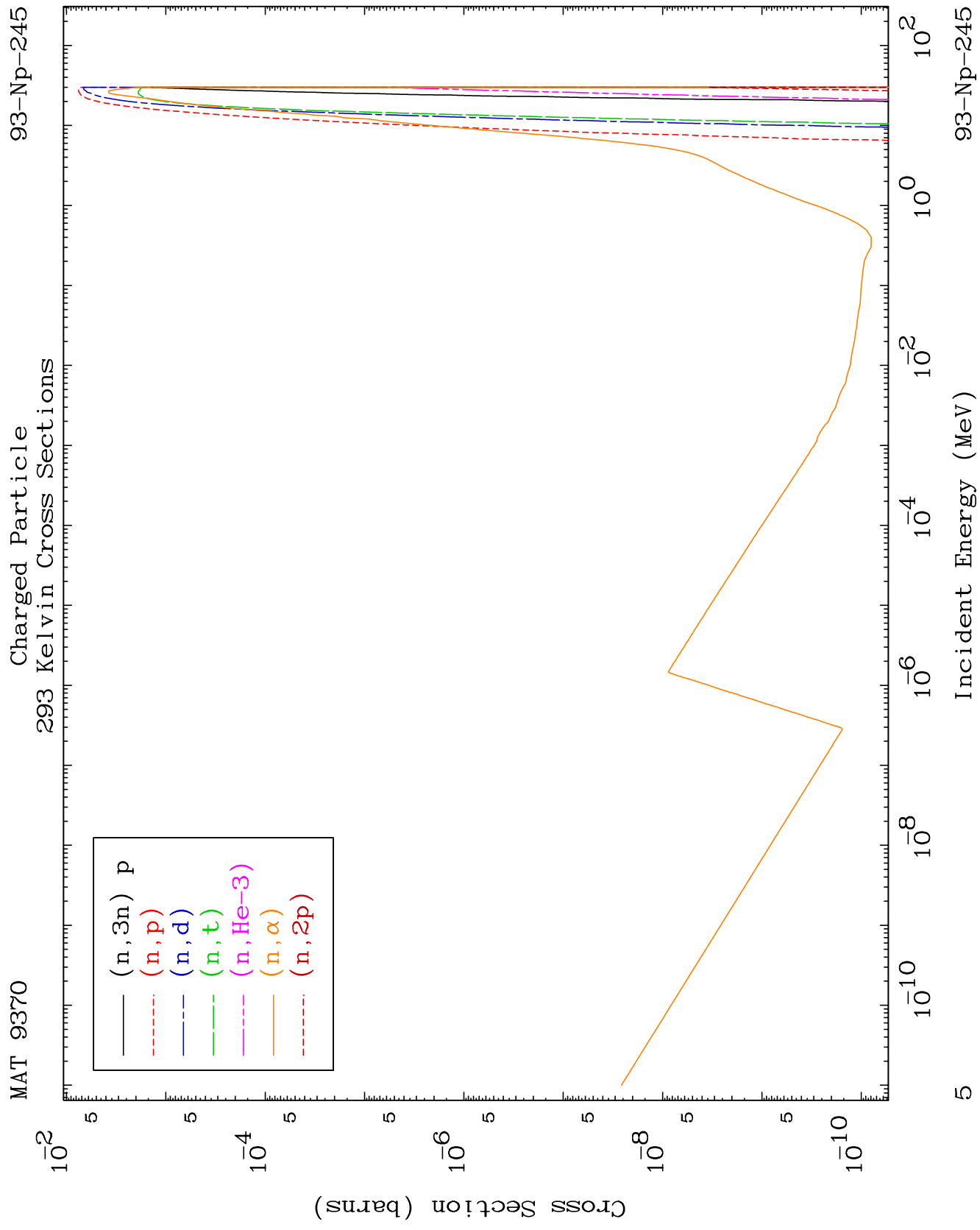


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Charged Particle
293 Kelvin Cross Sections

93-Np-245

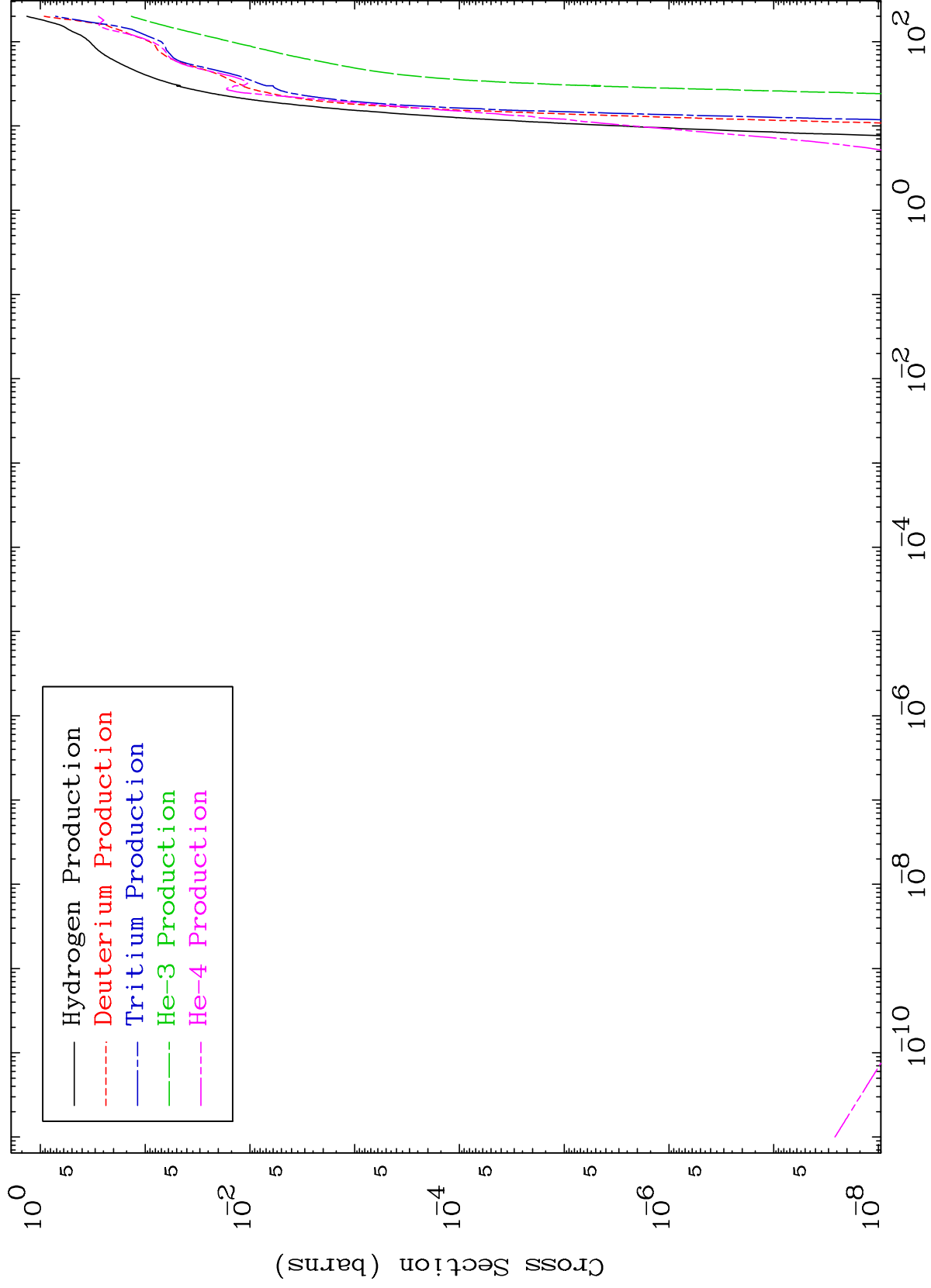




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Particle Production
293 Kelvin Cross Sections

93-Np-245

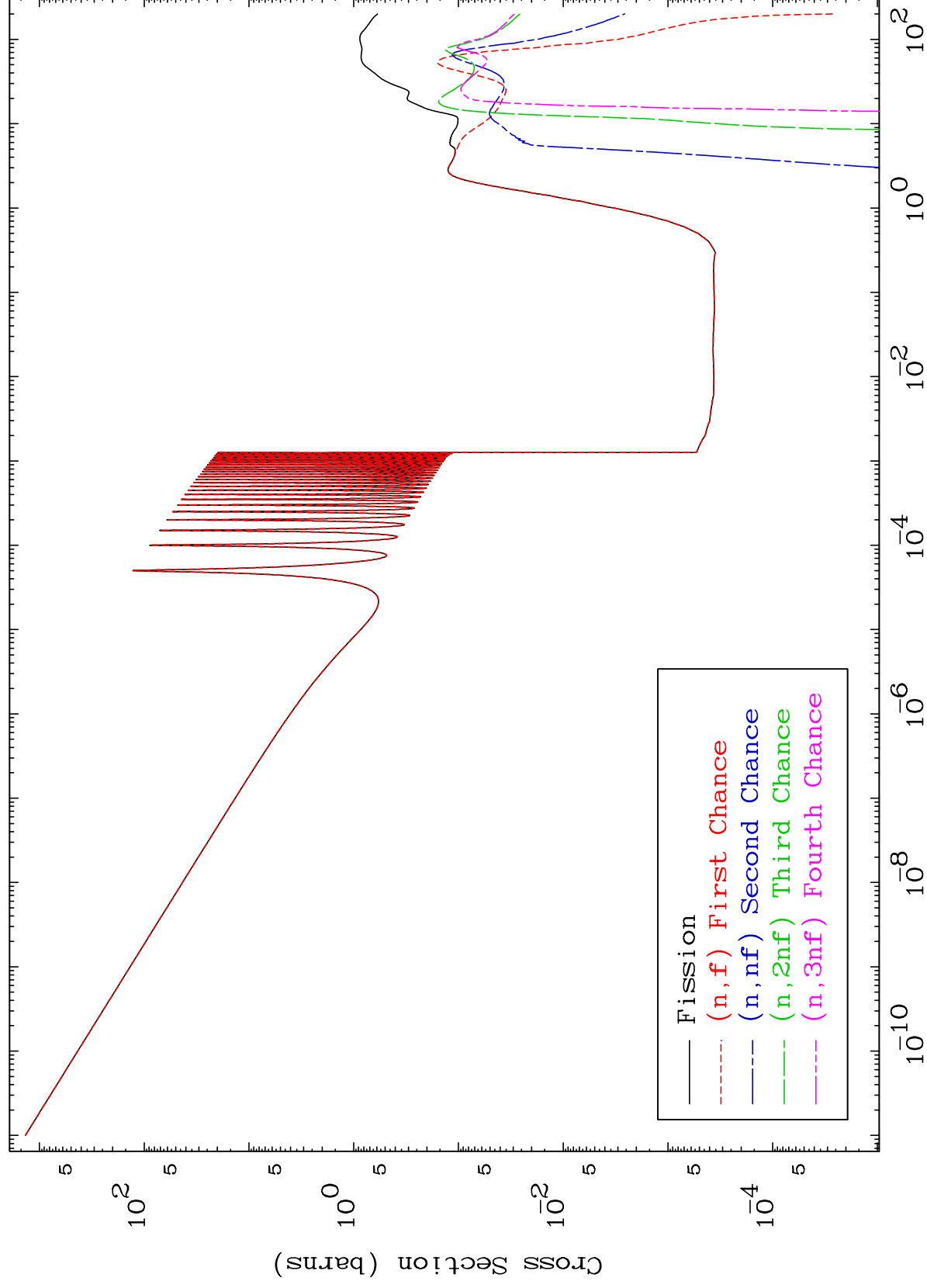


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Fission

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293 Kelvin Cross Sections

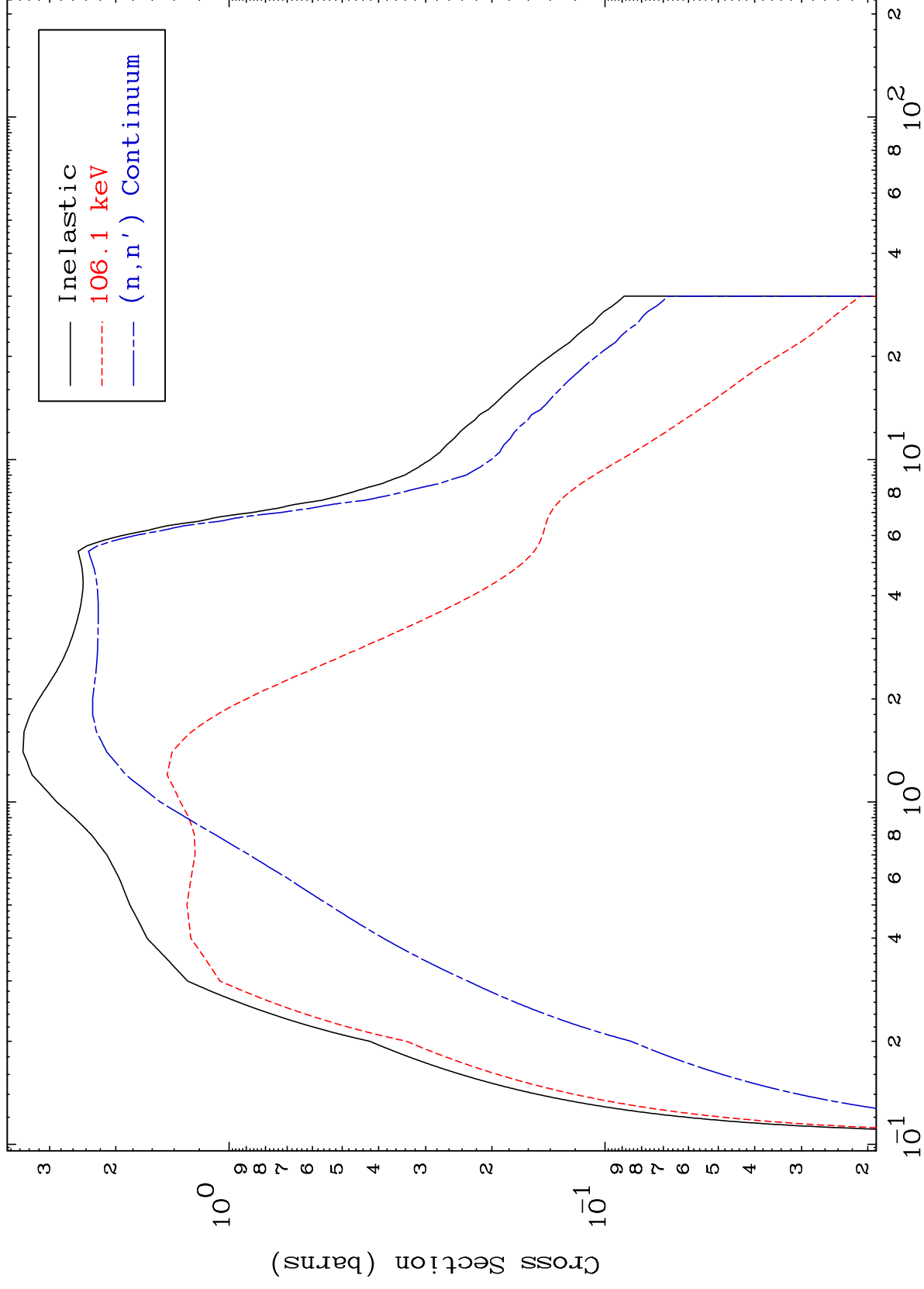


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(n,n') Level

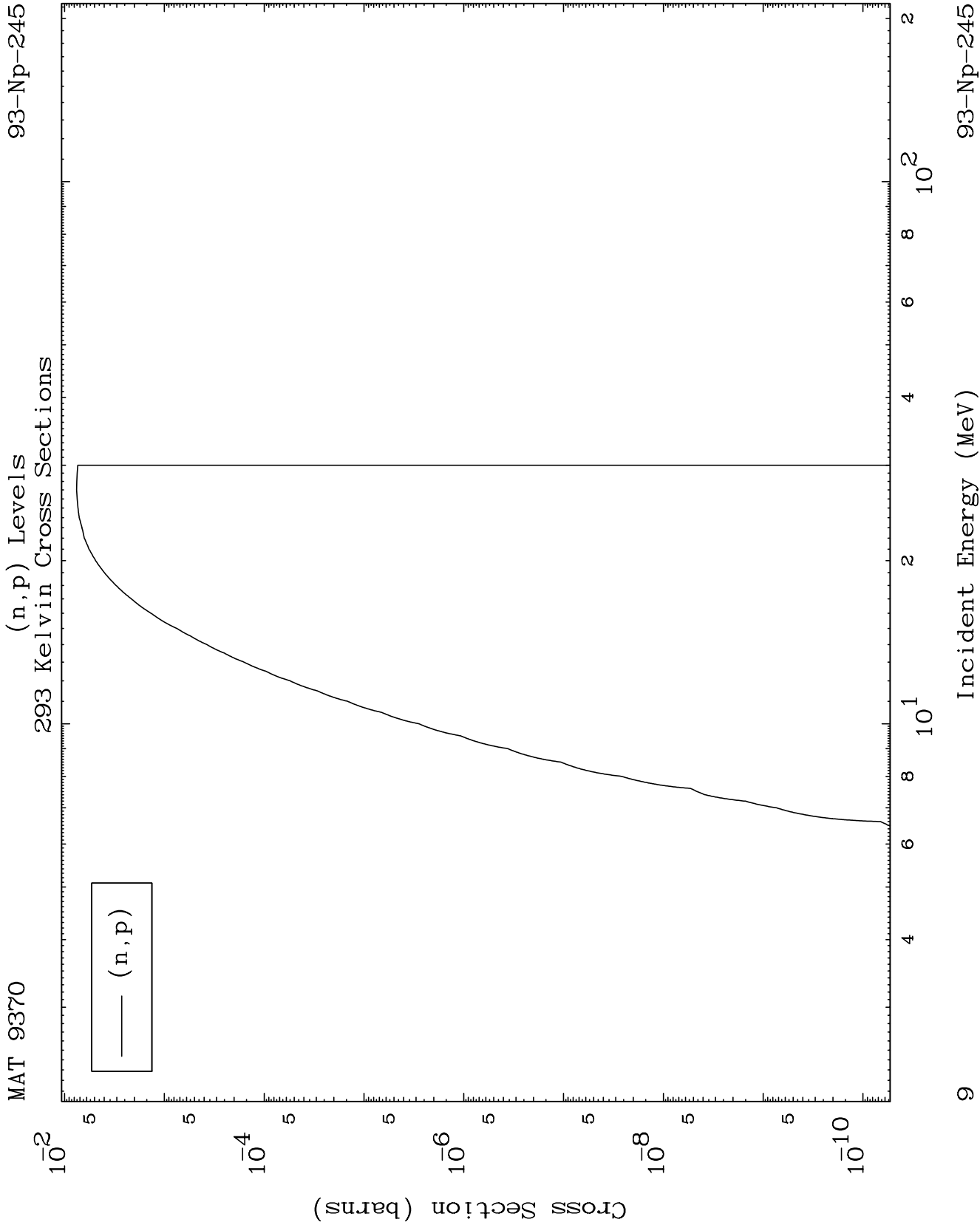
93-Np-245

293 Kelvin Cross Sections



Incident Energy (MeV)

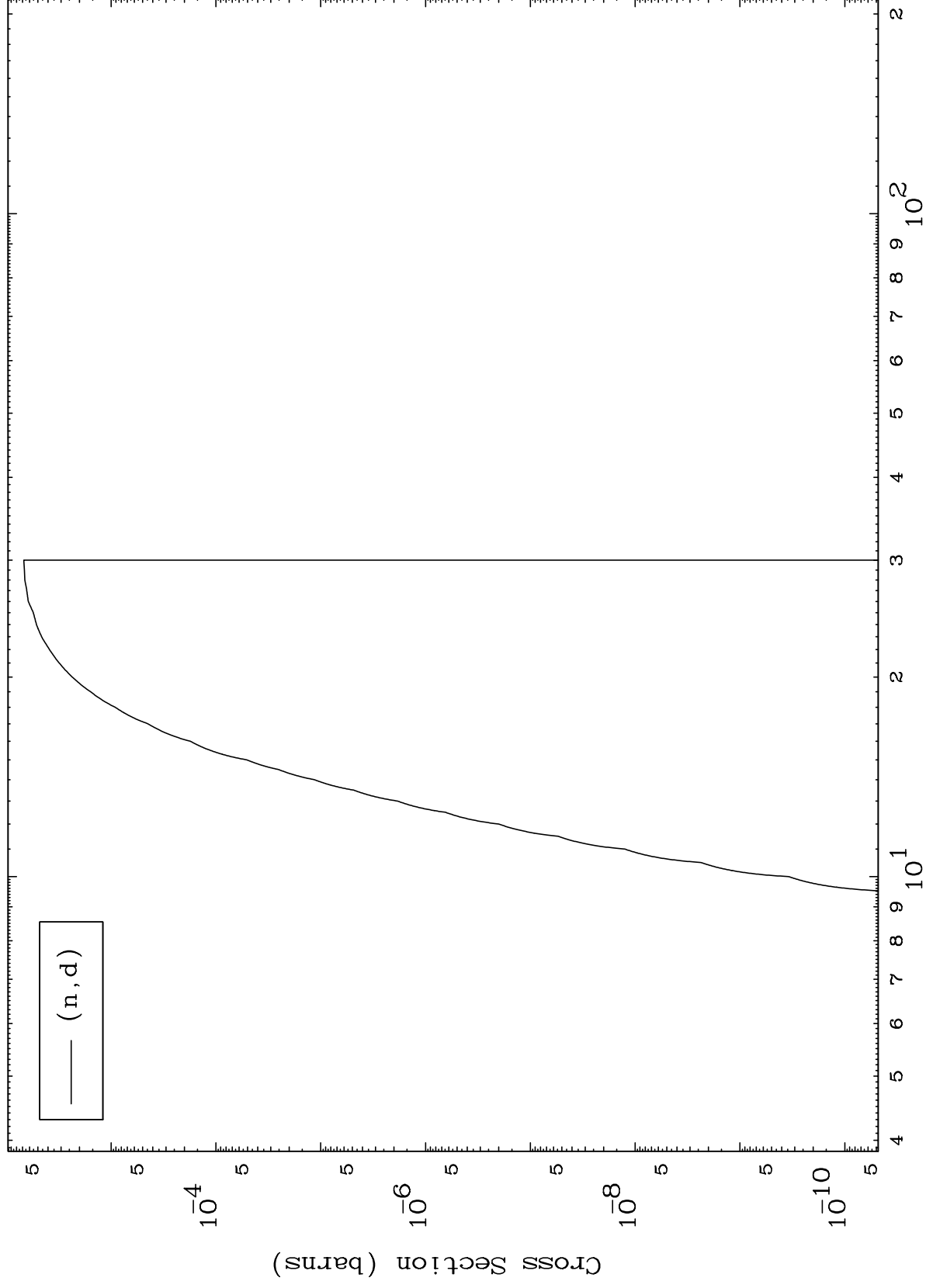
93-Np-245



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(n,d) Levels
293 Kelvin Cross Sections

93-Np-245



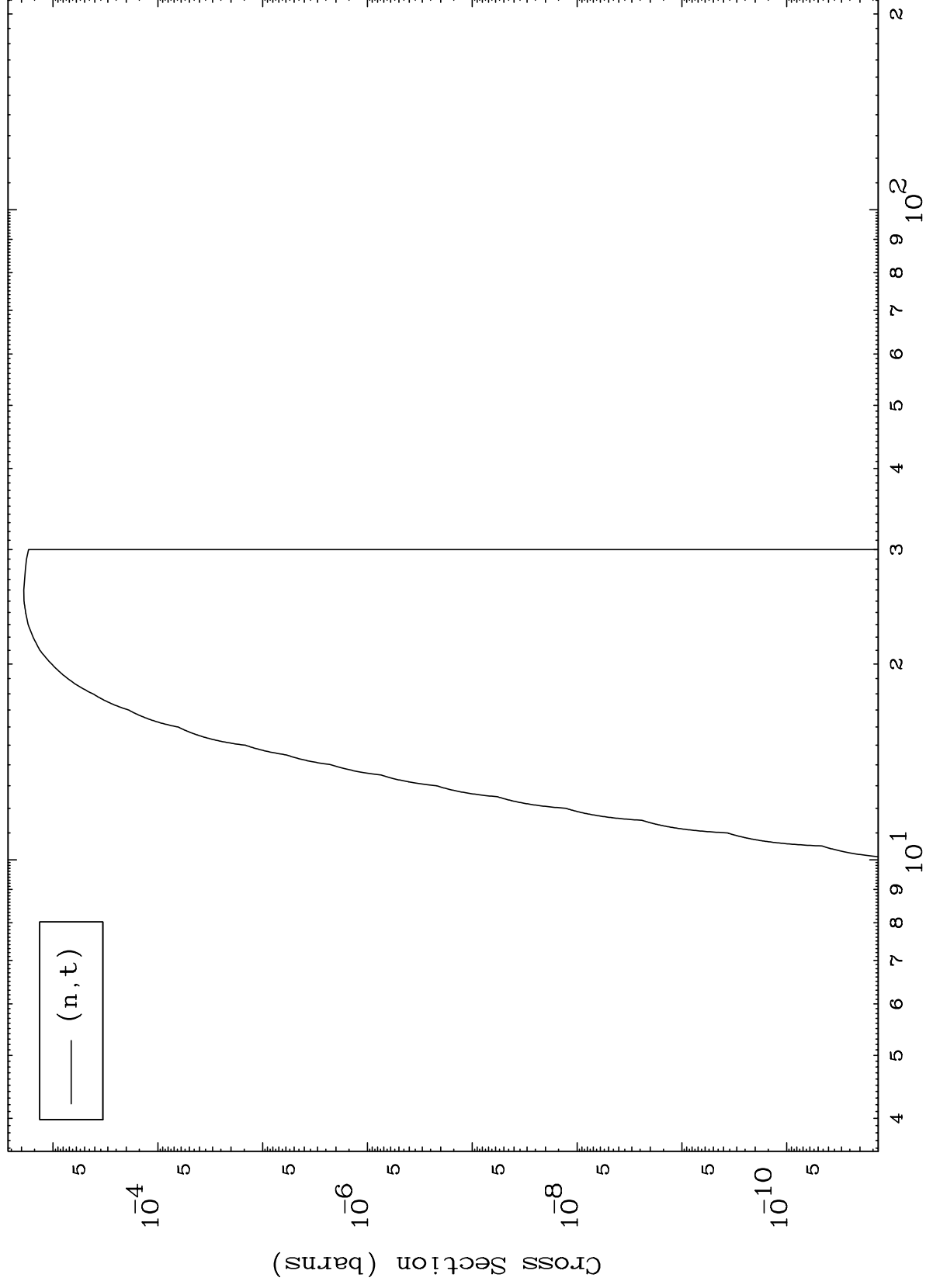
Incident Energy (MeV)

93-Np-245

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93-Np-245

(n,t) Levels
293 Kelvin Cross Sections



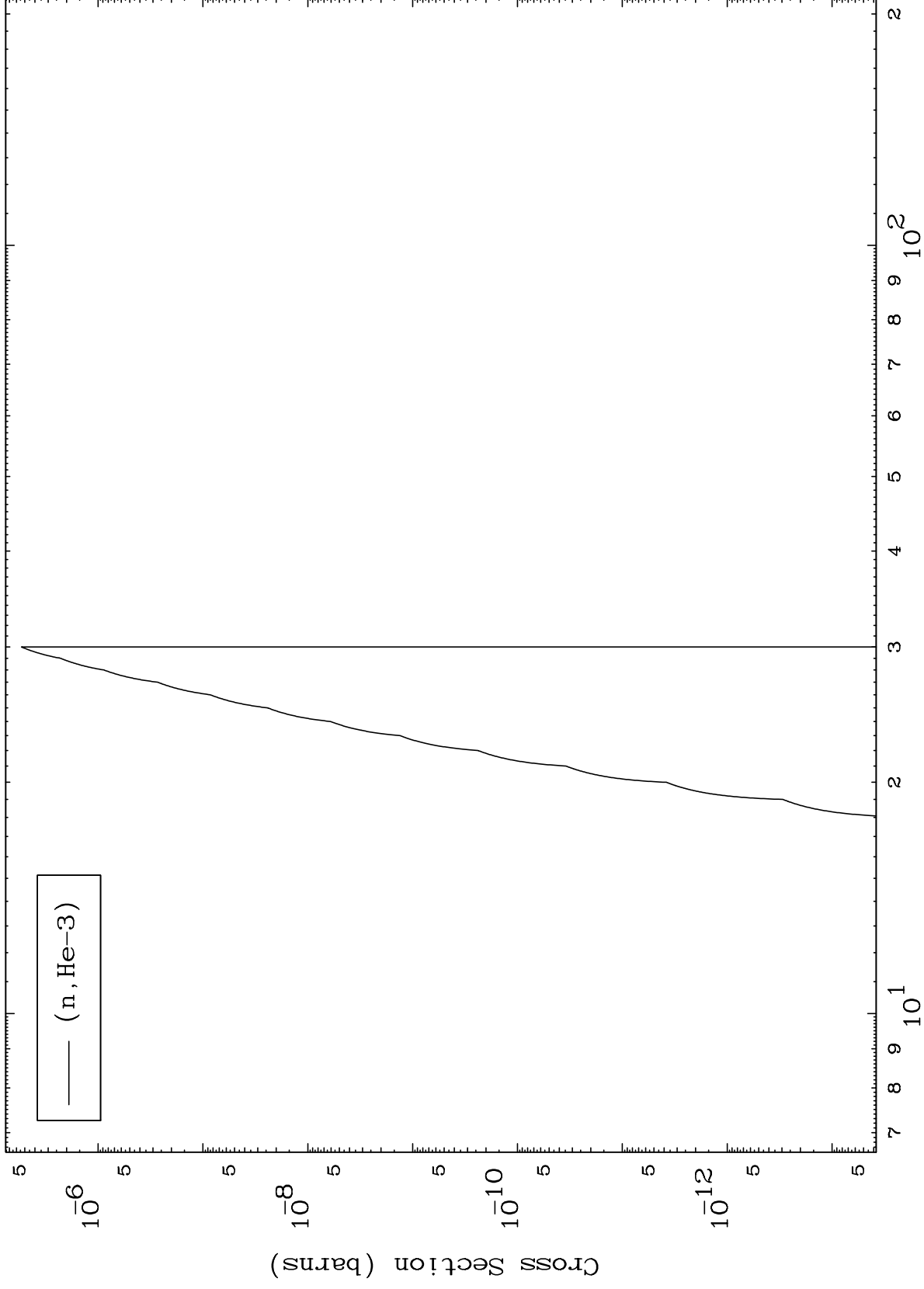
11

93-Np-245

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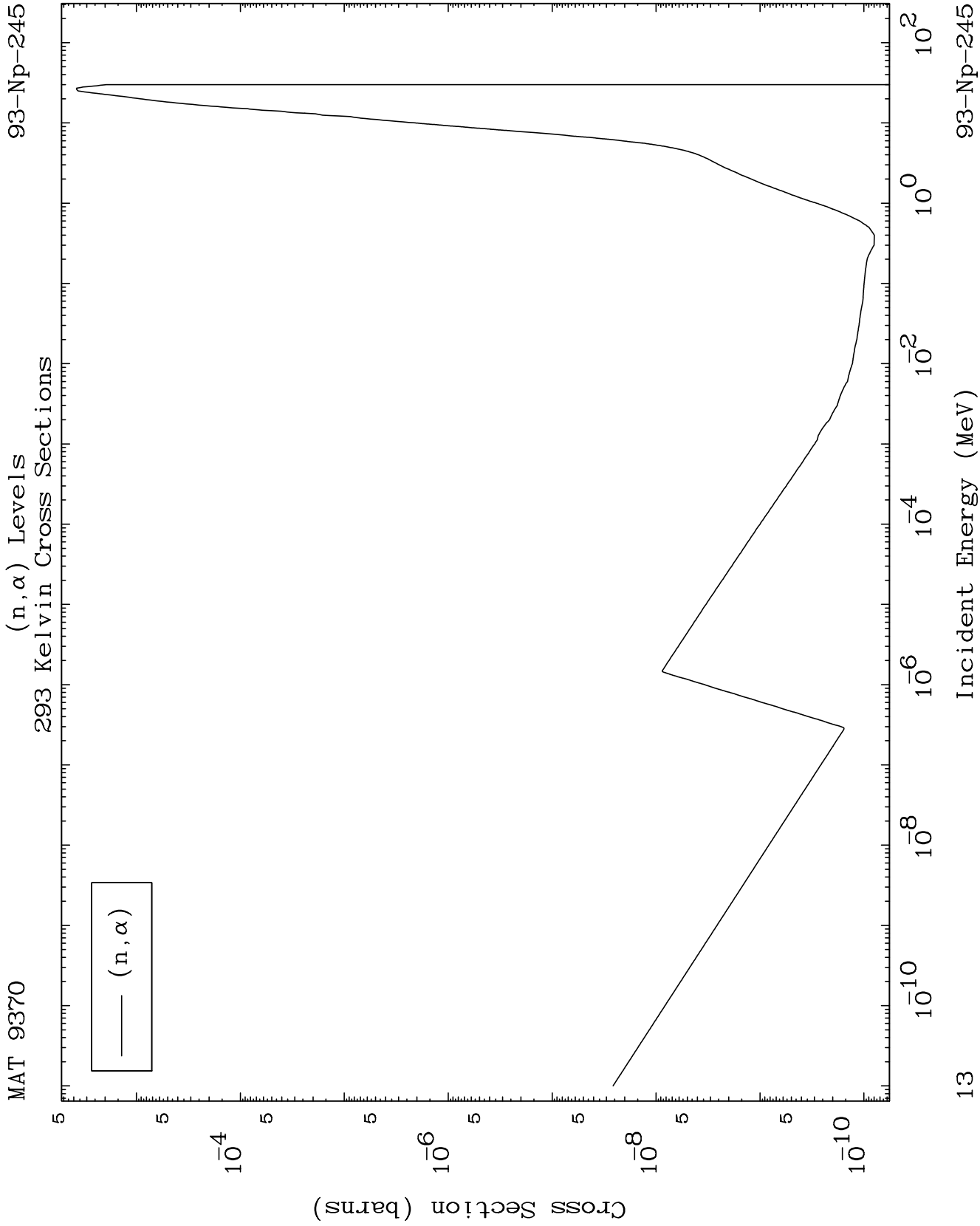
(n,He3) Levels
293 Kelvin Cross Sections

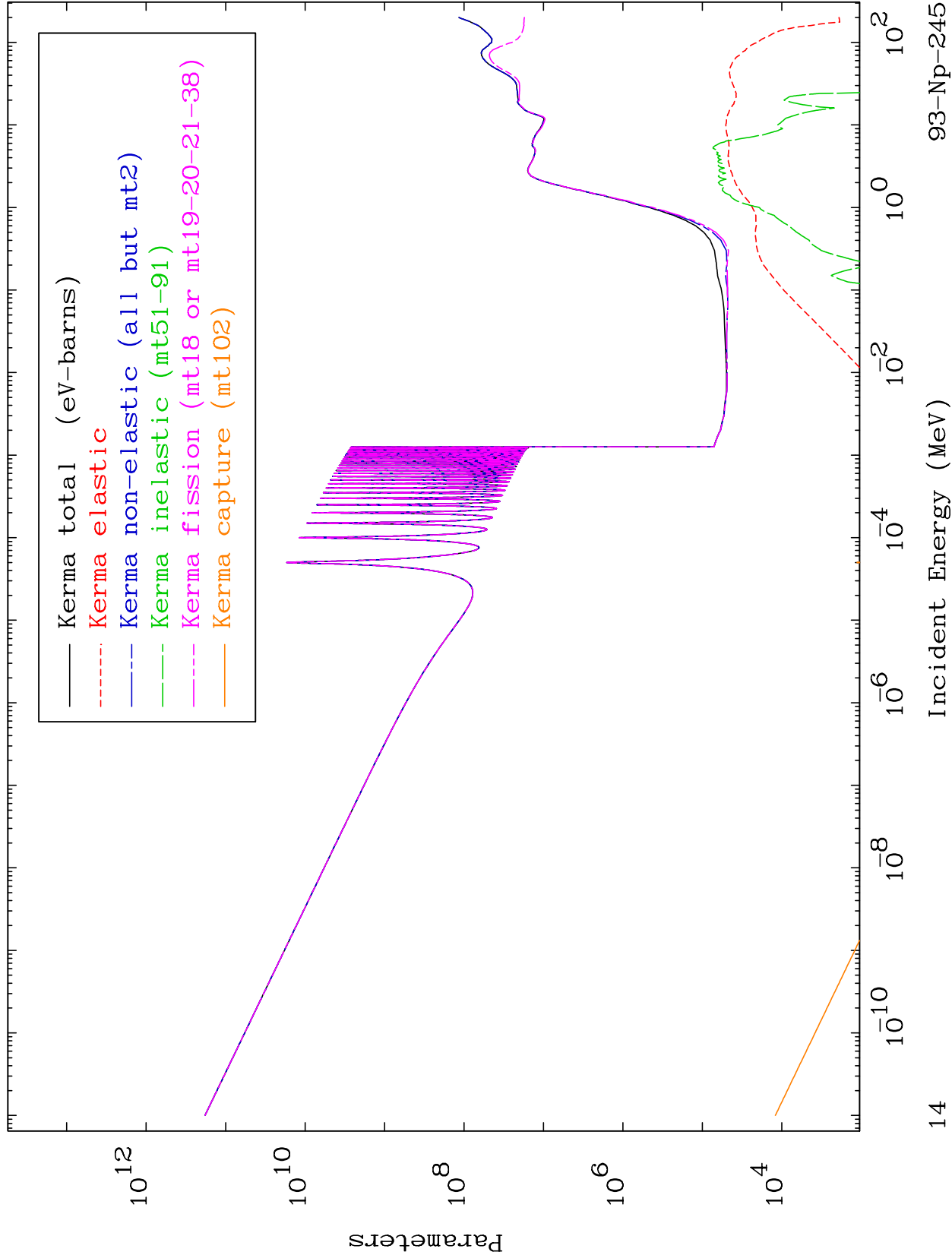
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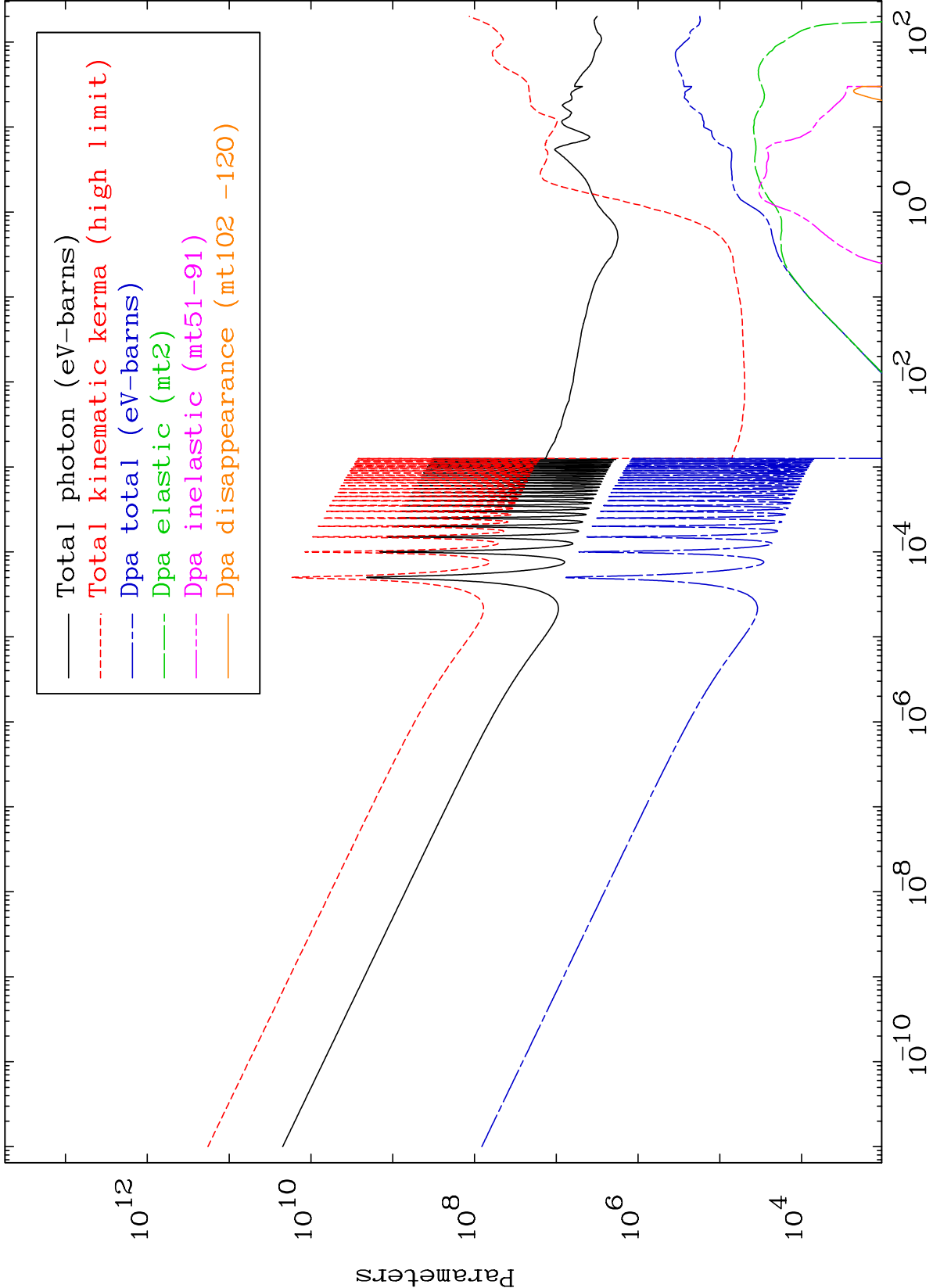


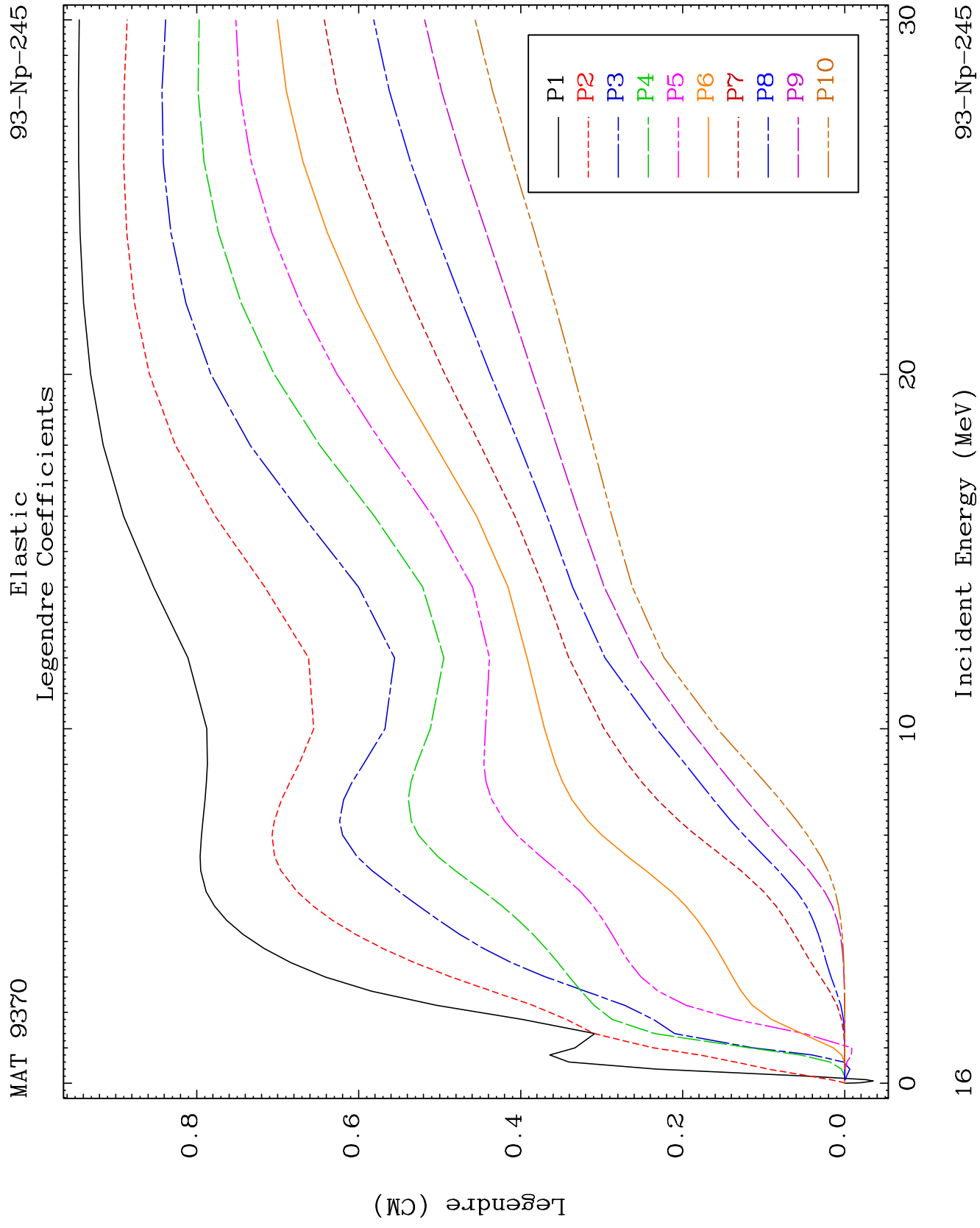
Incident Energy (MeV)

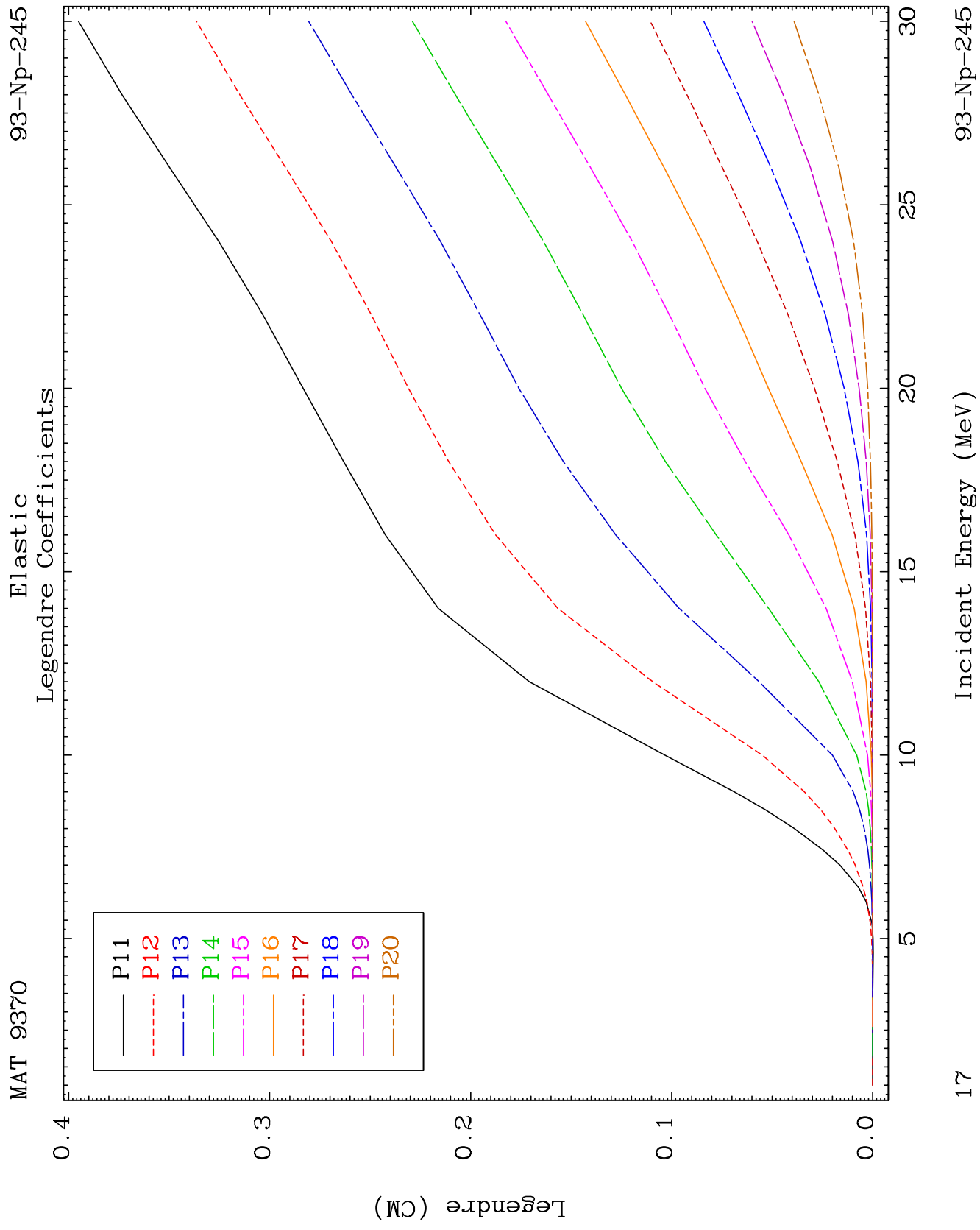
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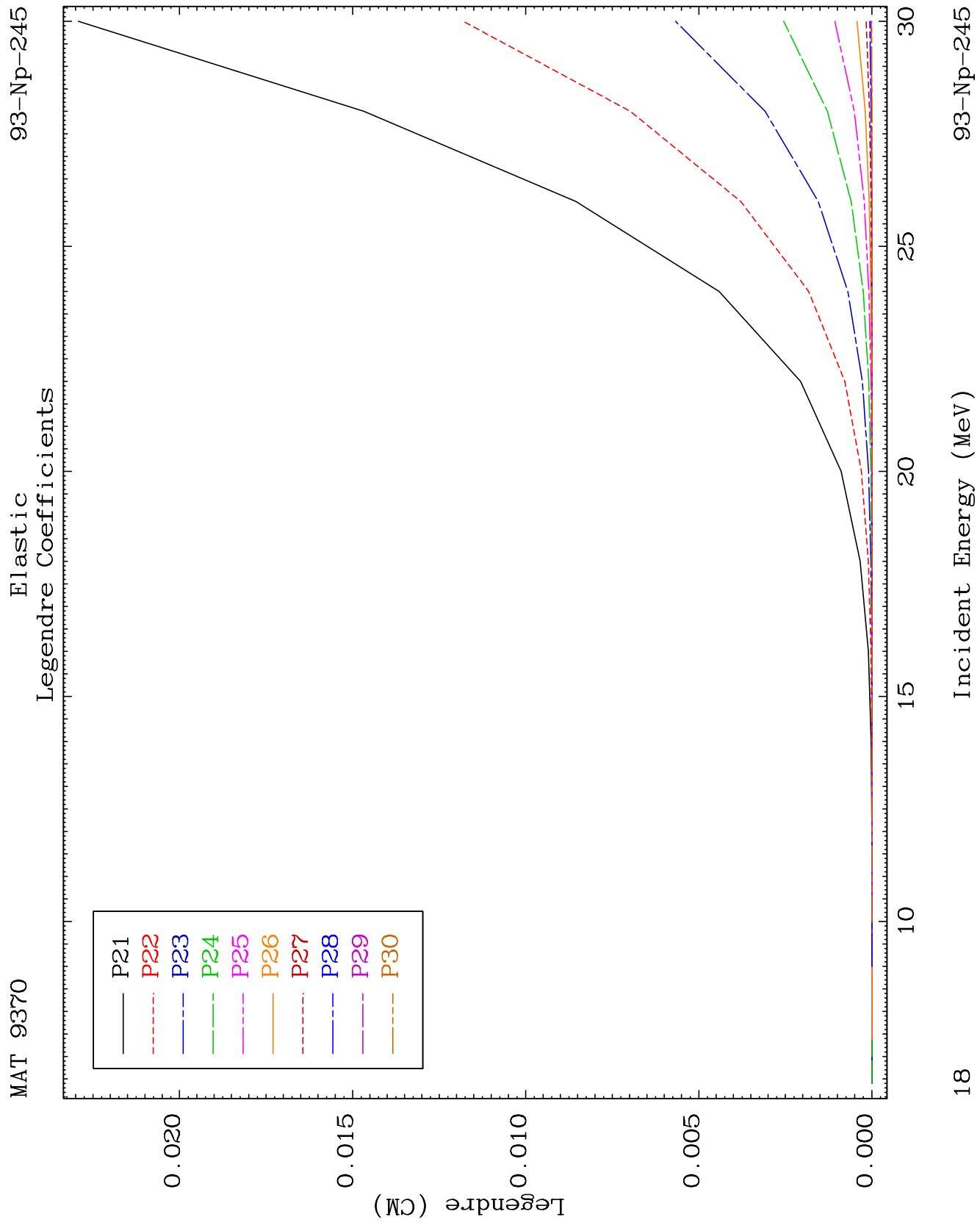


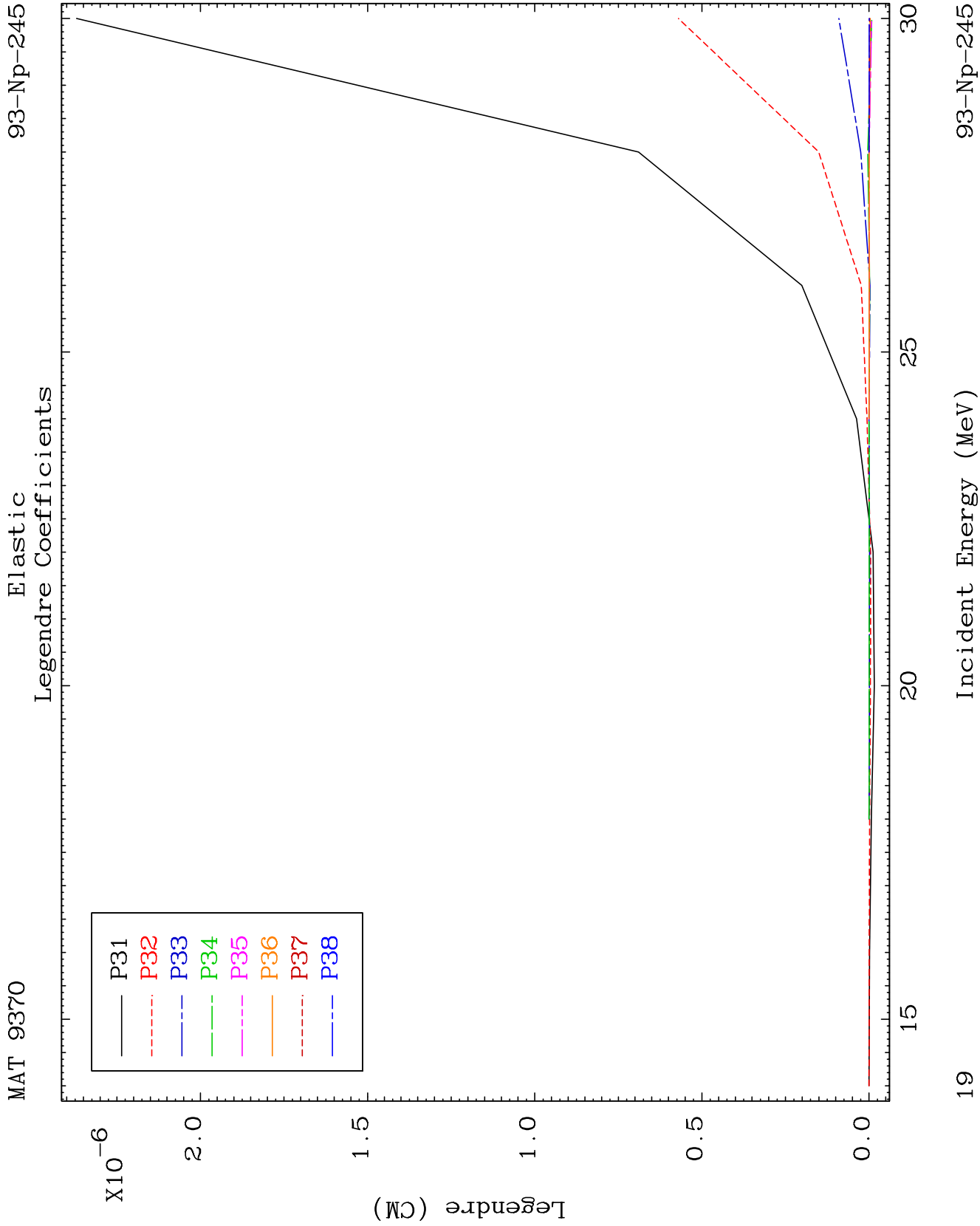




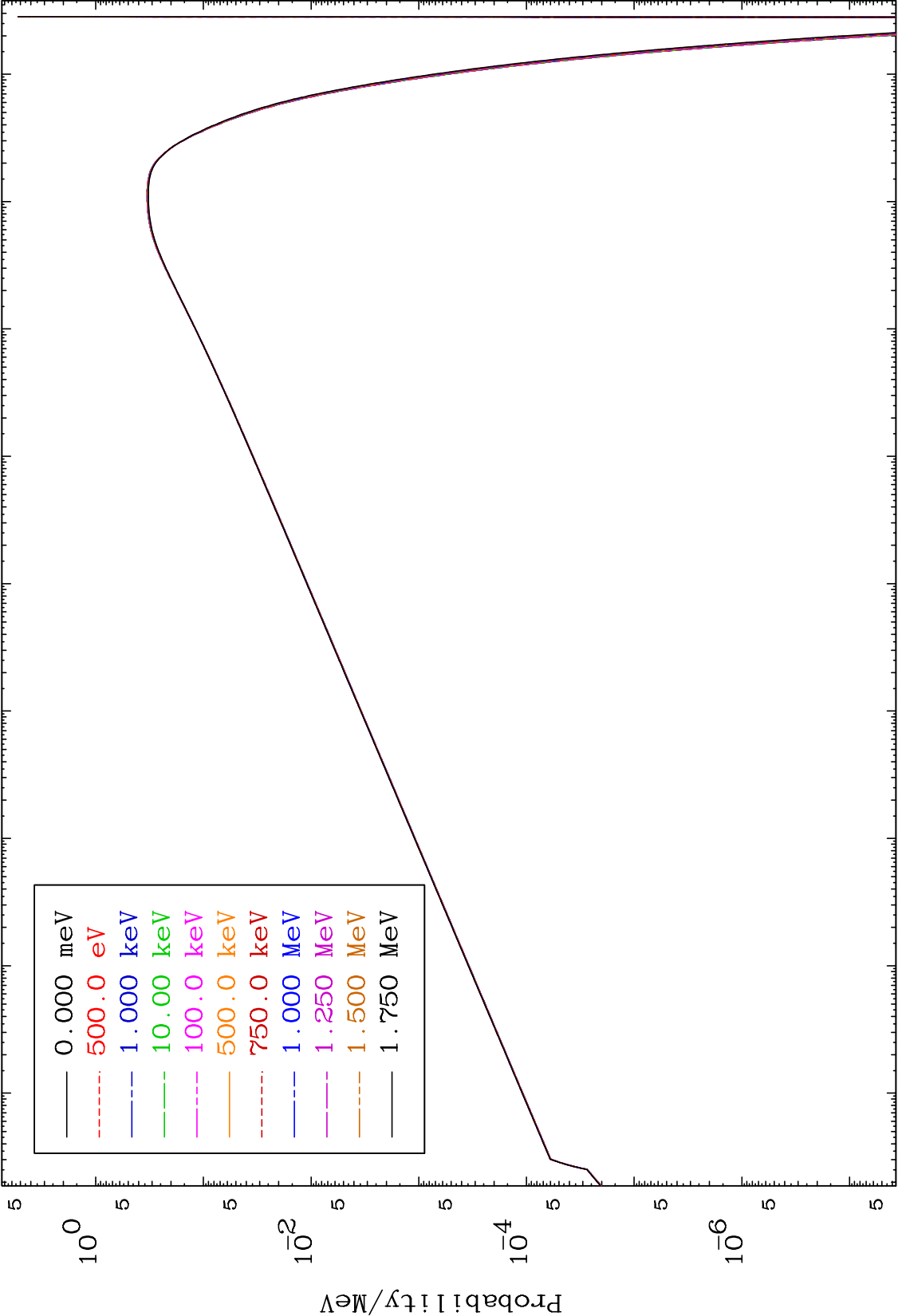




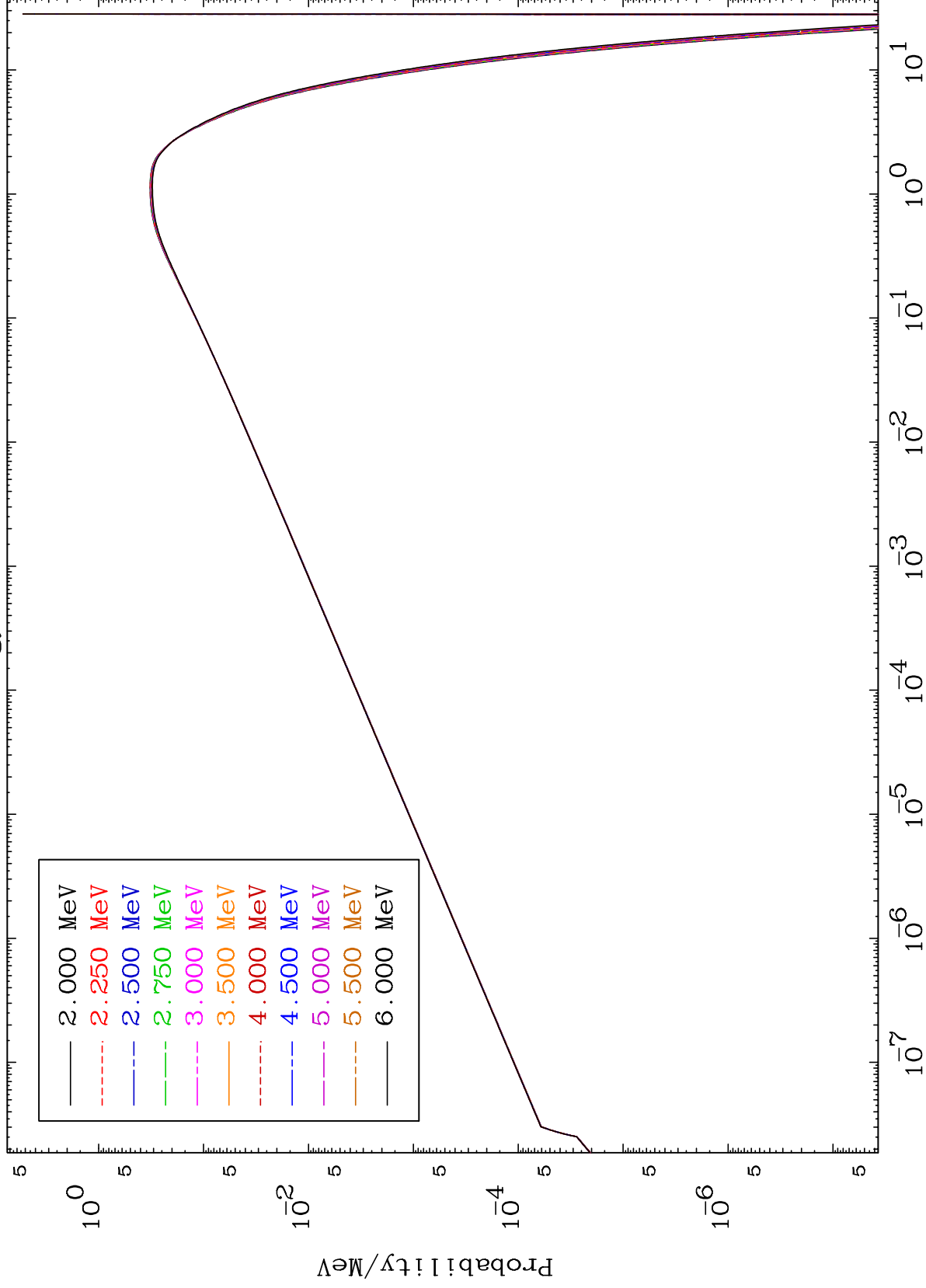




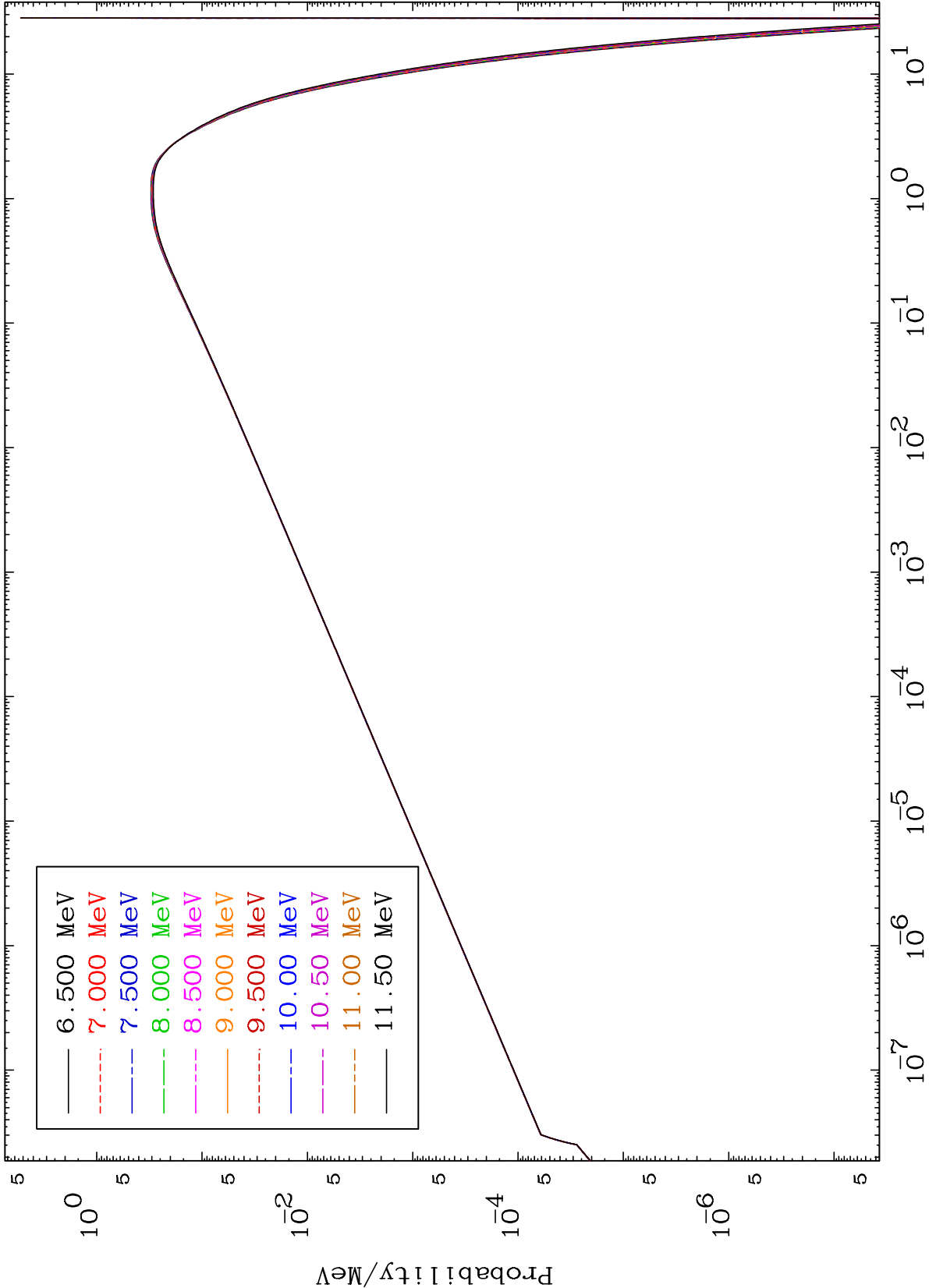
Energy Distribution



Energy Distribution



Energy Distribution



MAT 9370

Fission

93-Np-245

Energy Distribution

