

Program EVALPLOT  
(Version 2015-2)

by

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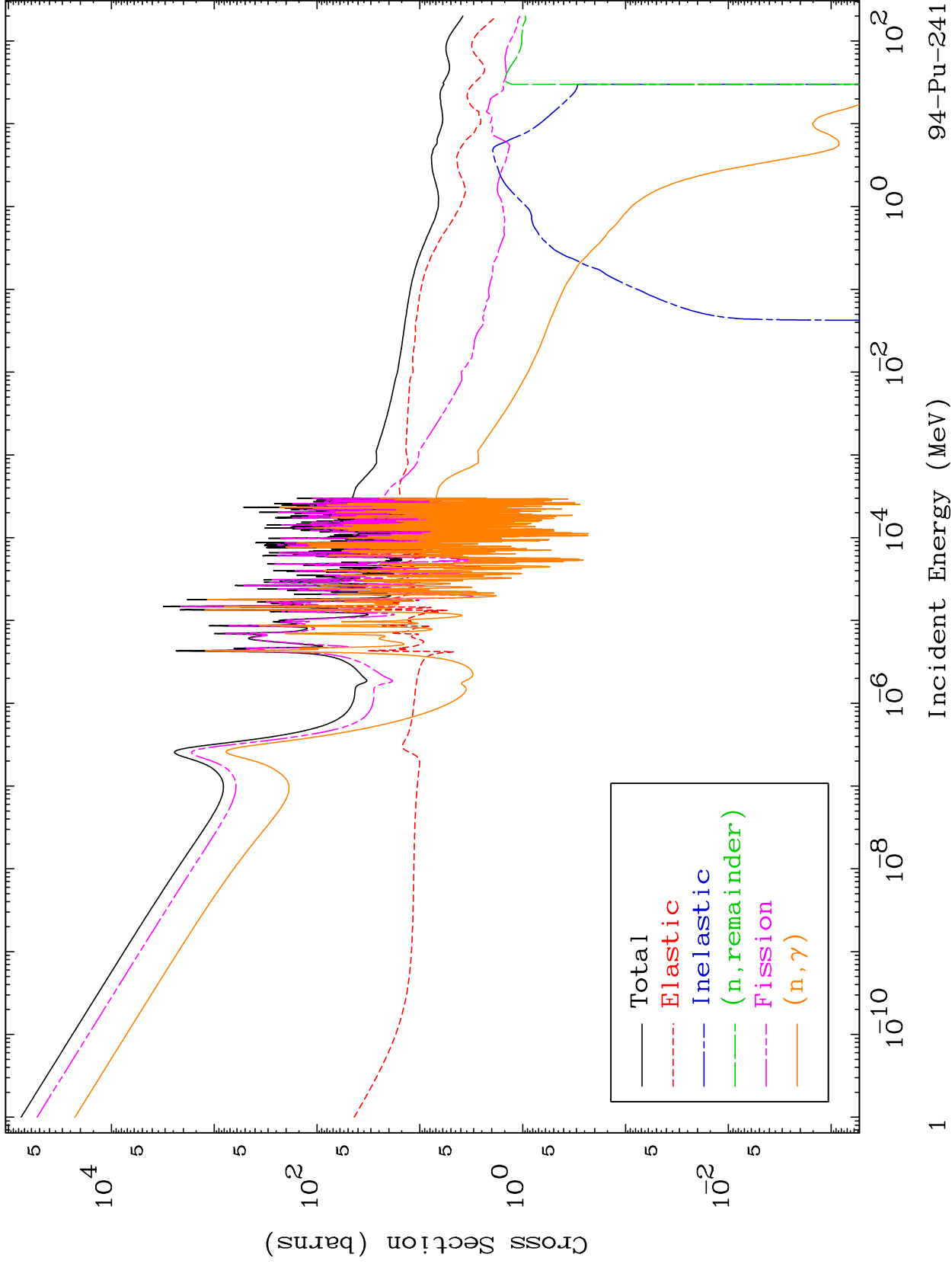
Press Mouse Button to Start

MAT 9443

Major

293 Kelvin Cross Sections

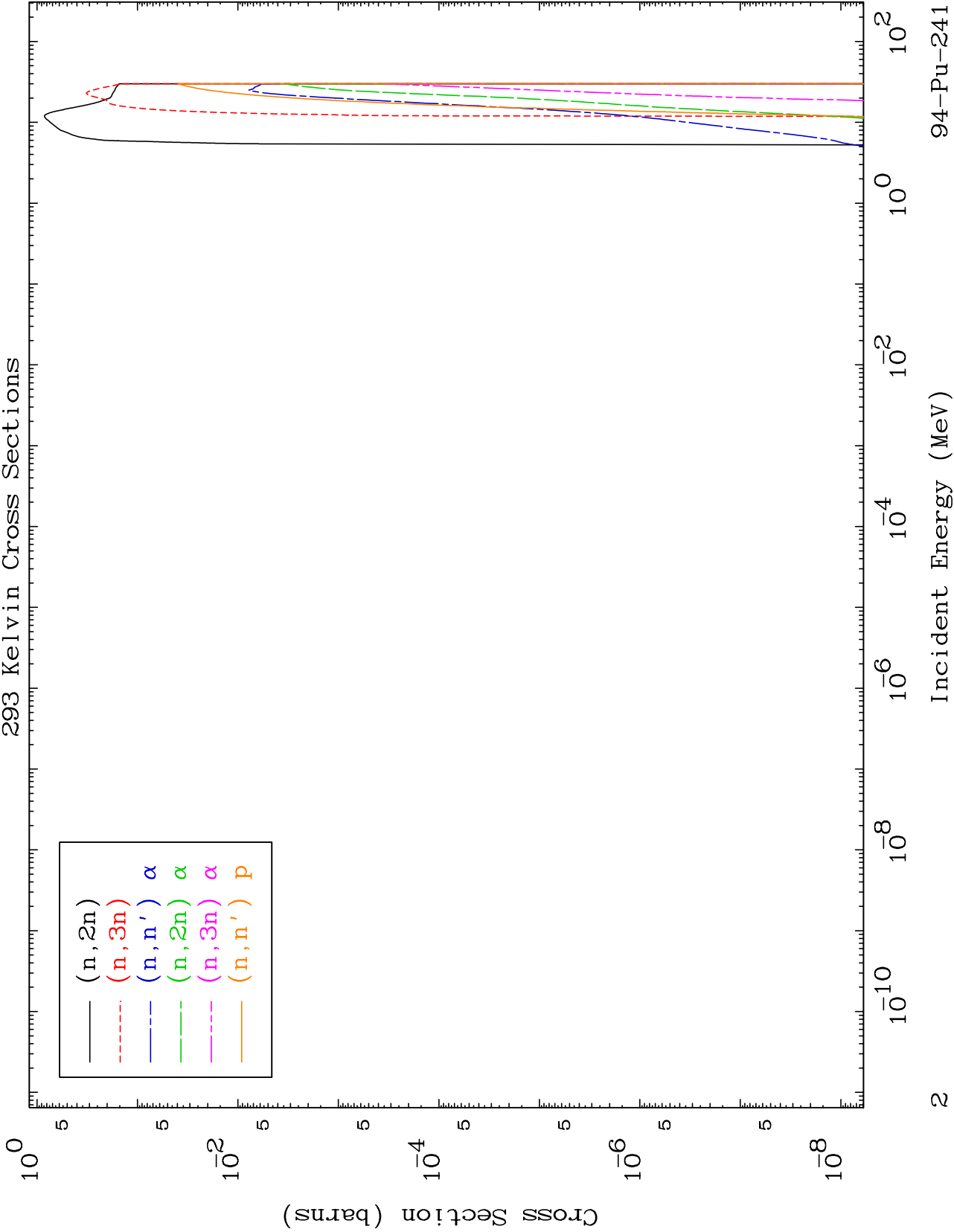
94-Pu-241



MAT 9443

Neutron Production  
293 Kelvin Cross Sections

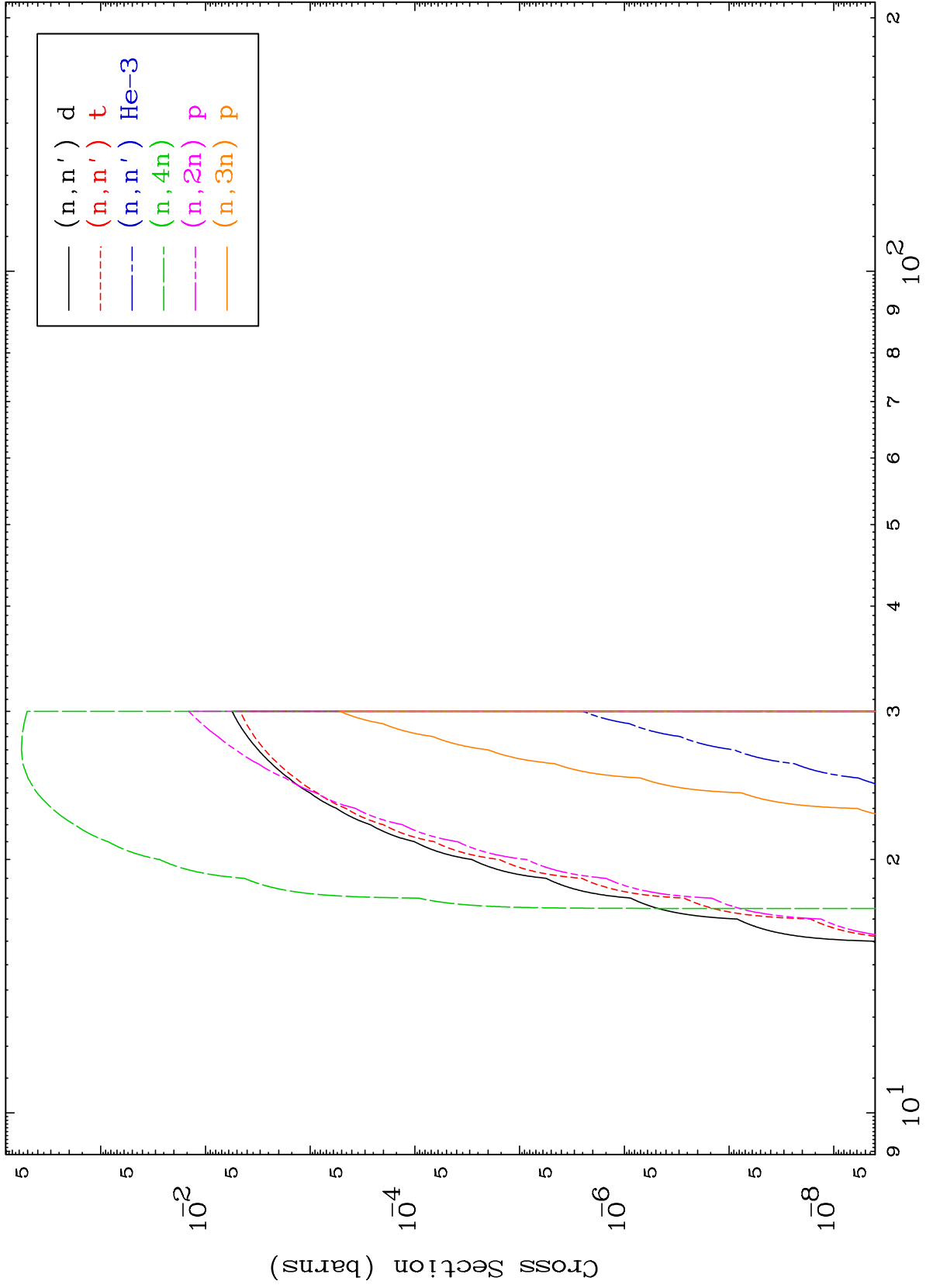
94-Pu-241



MAT 9443

Neutron Production  
293 Kelvin Cross Sections

94-Pu-241



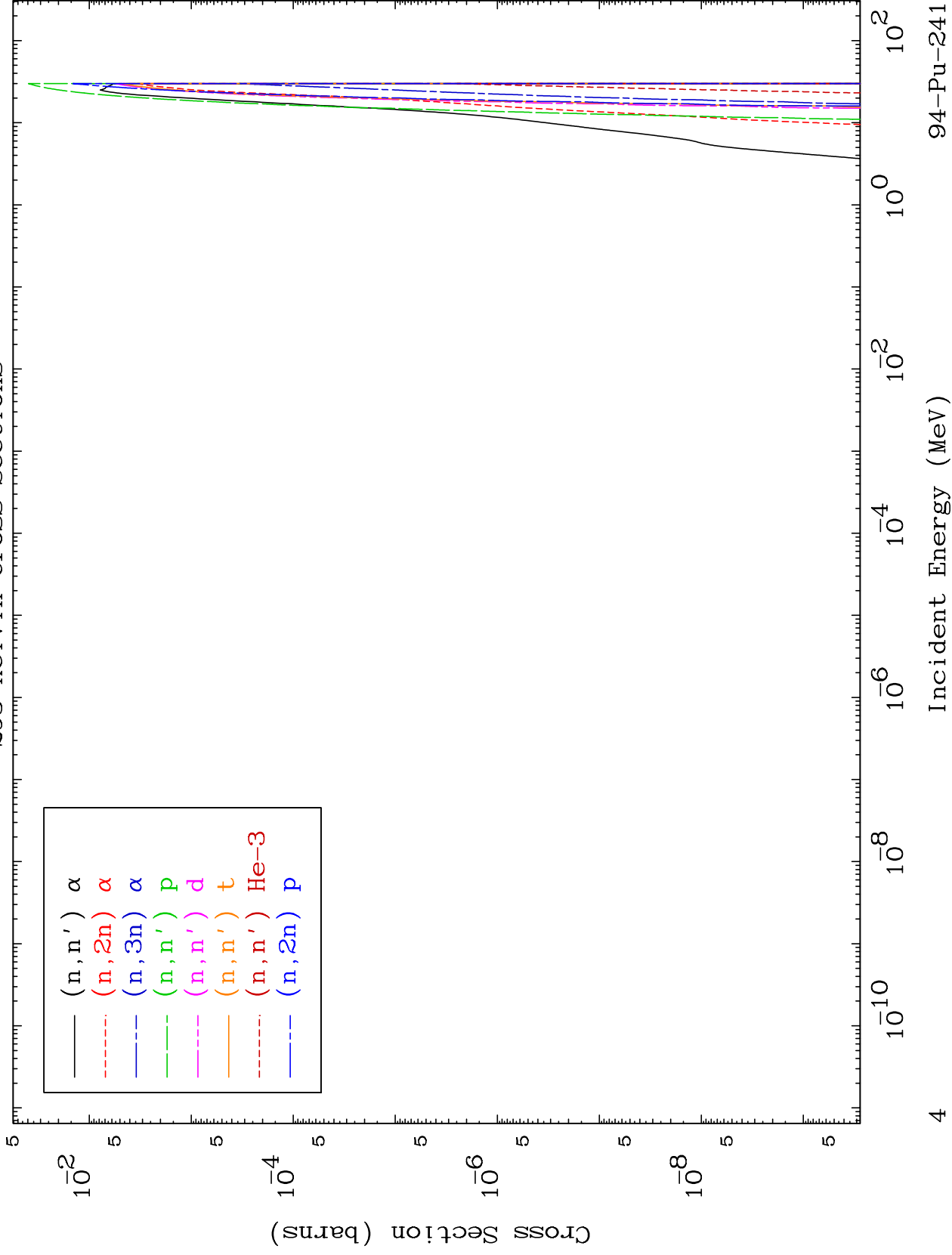
Incident Energy (MeV)

94-Pu-241

MAT 9443

Charged Particle  
293 Kelvin Cross Sections

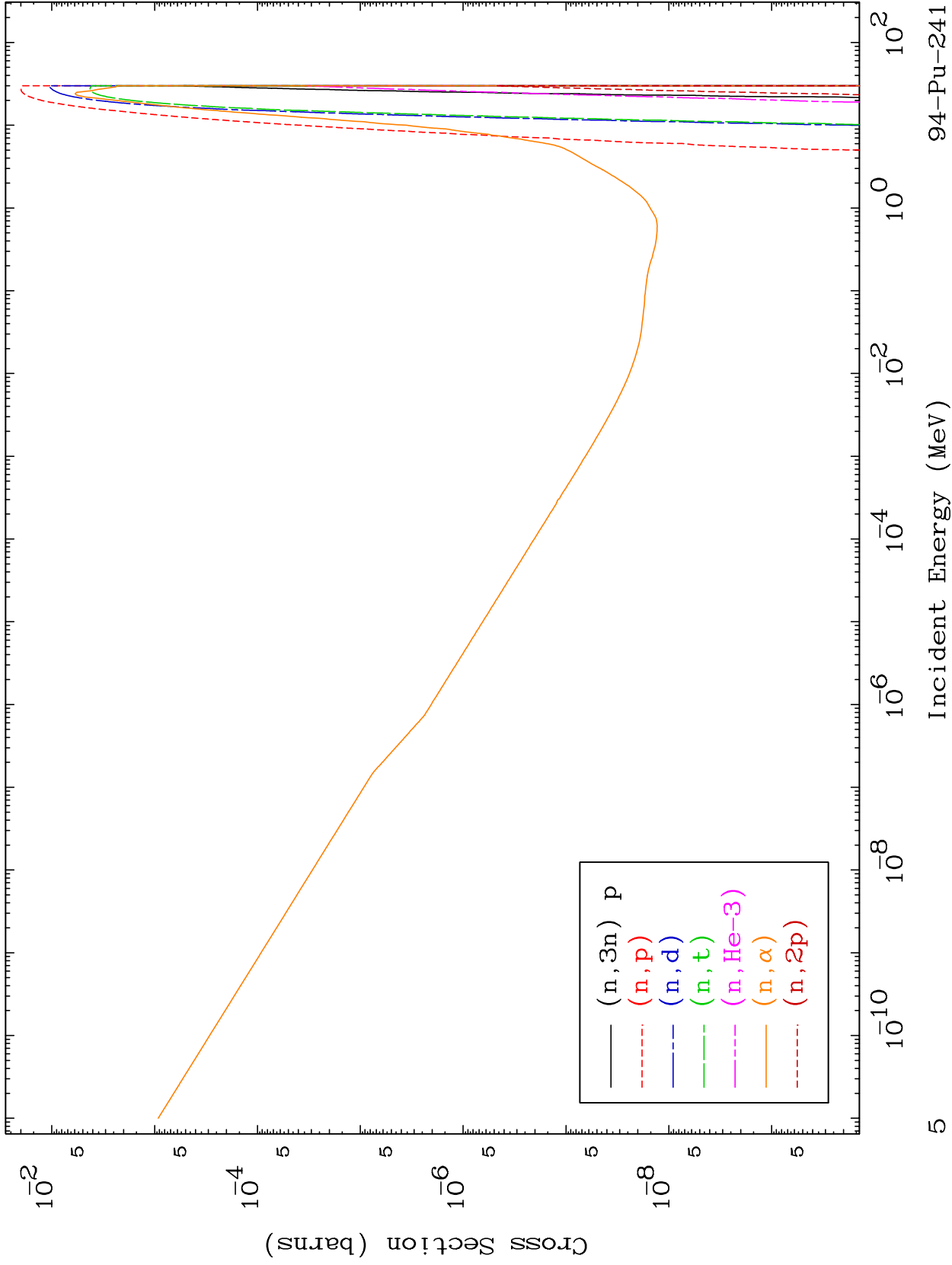
94-Pu-241



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Charged Particle  
293 Kelvin Cross Sections

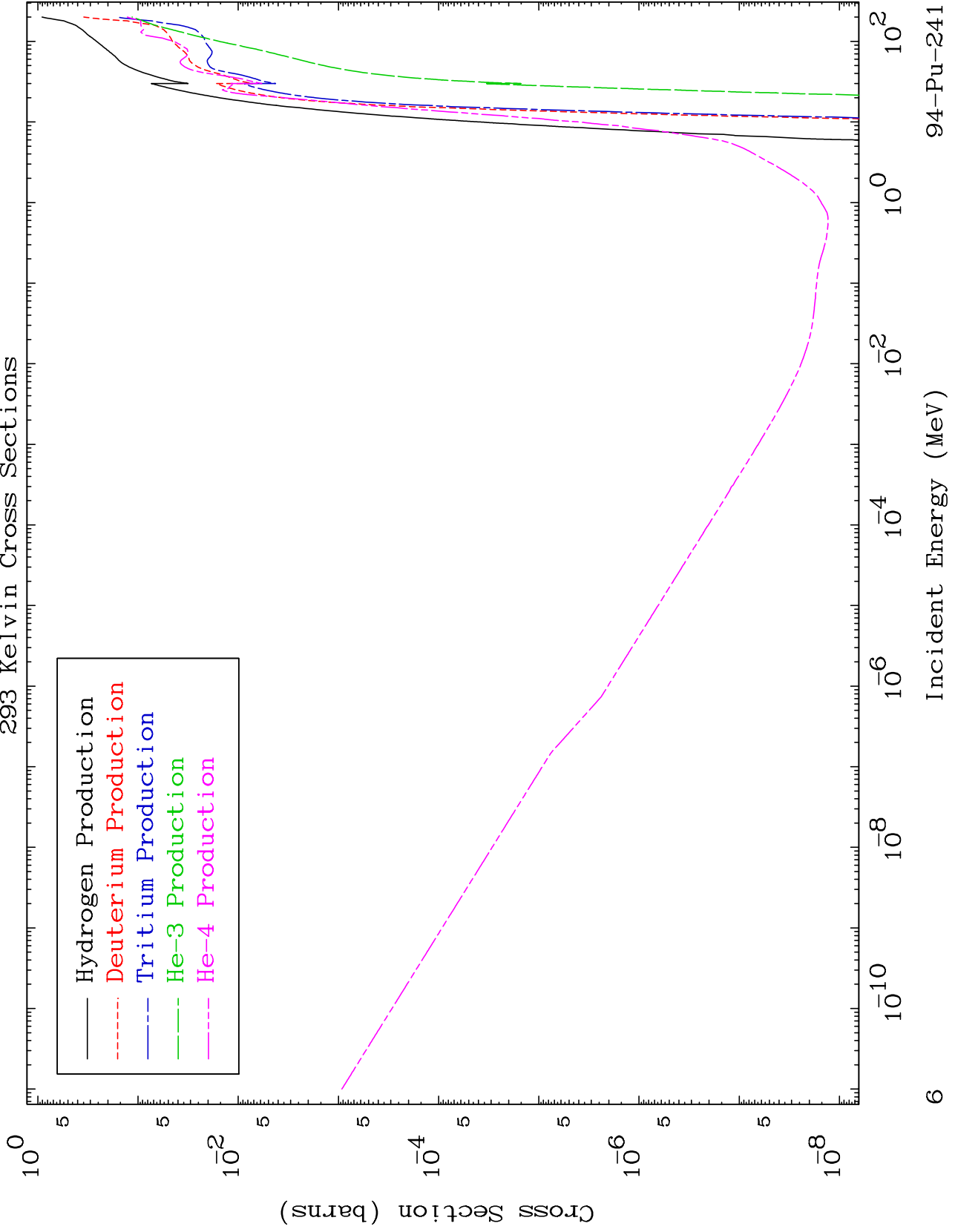
94-Pu-241



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Particle Production  
293 Kelvin Cross Sections

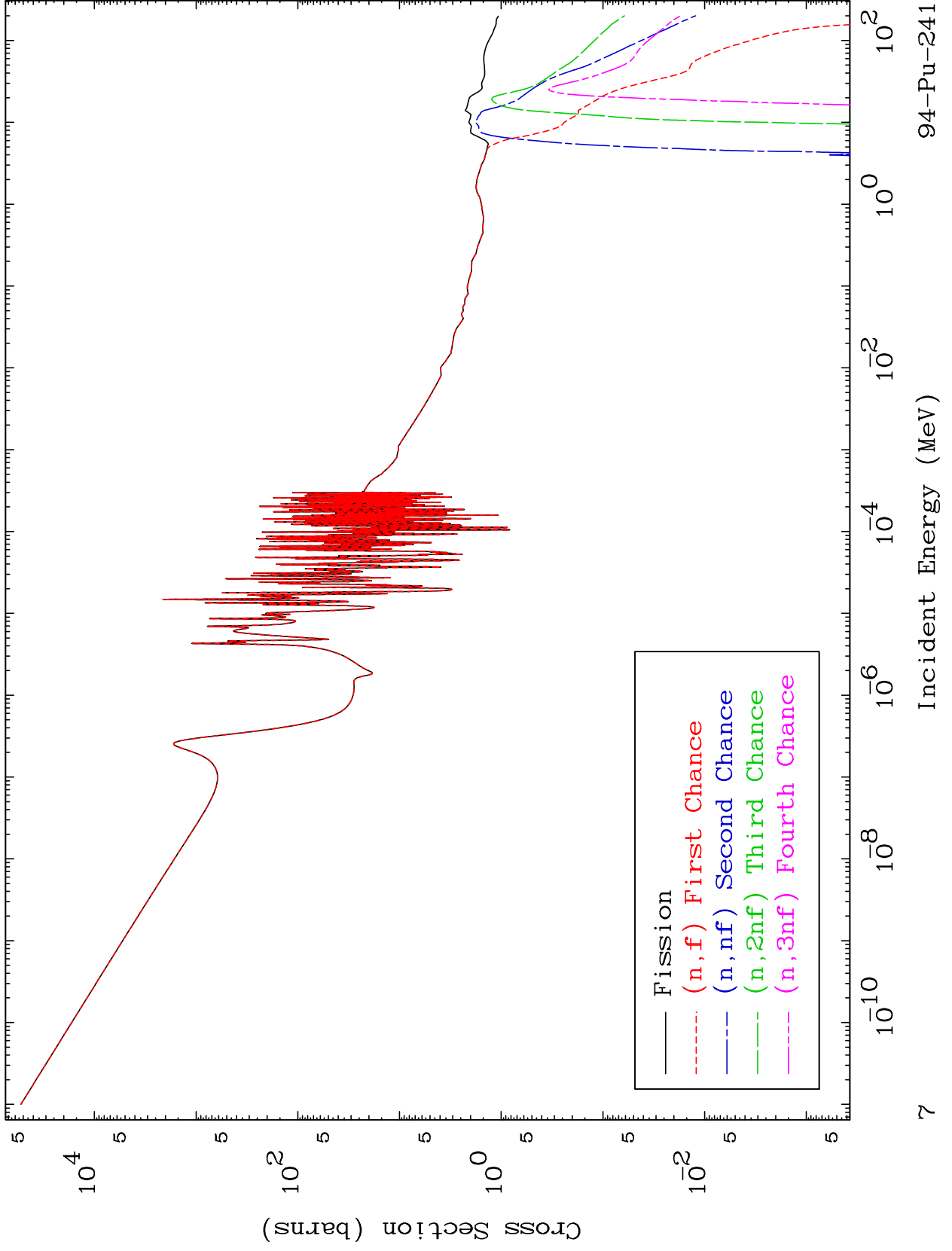
94-Pu-241



MAT 9443

Fission  
293 Kelvin Cross Sections

94-Pu-241

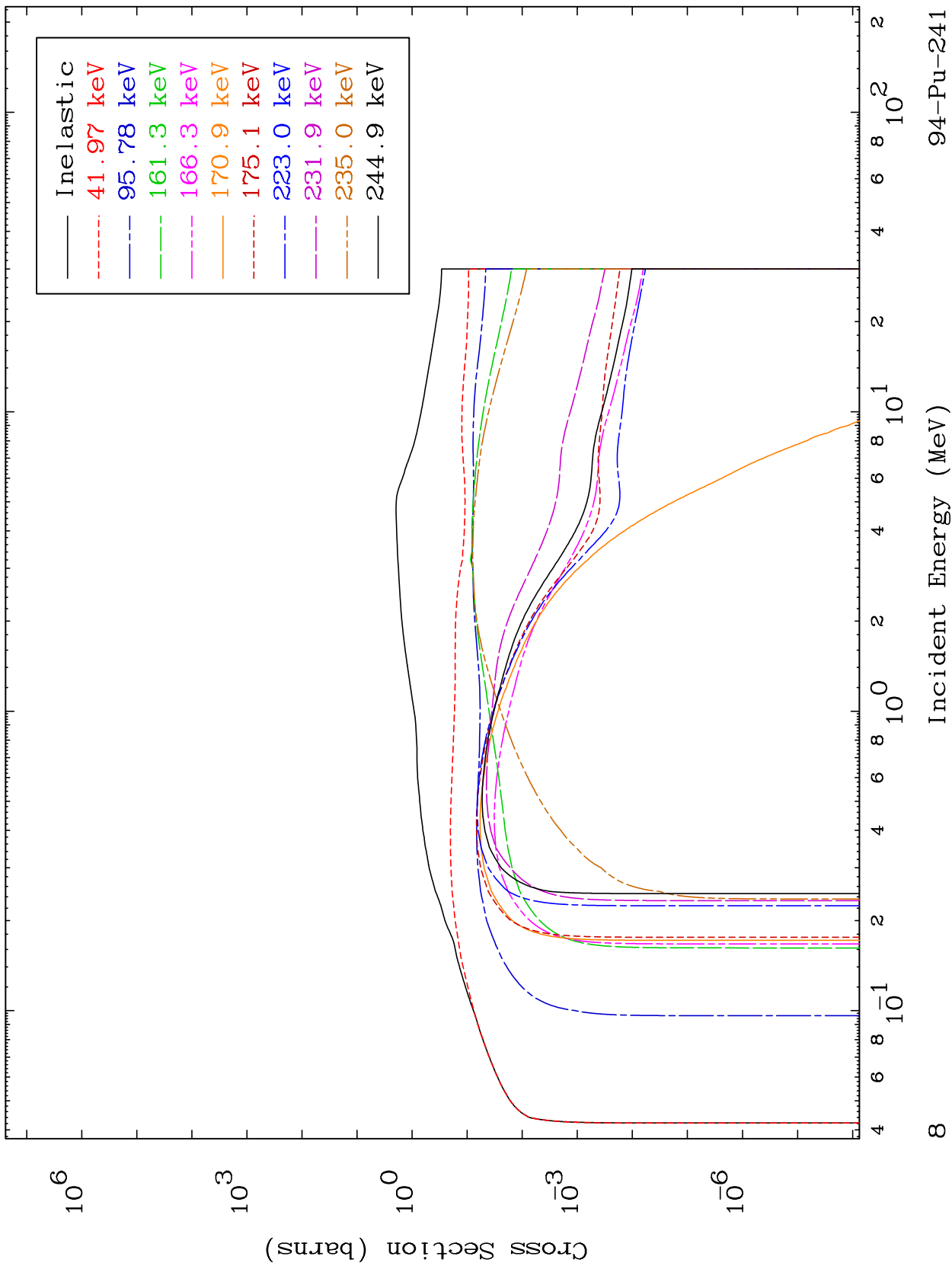




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(n,n') Level  
293 Kelvin Cross Sections

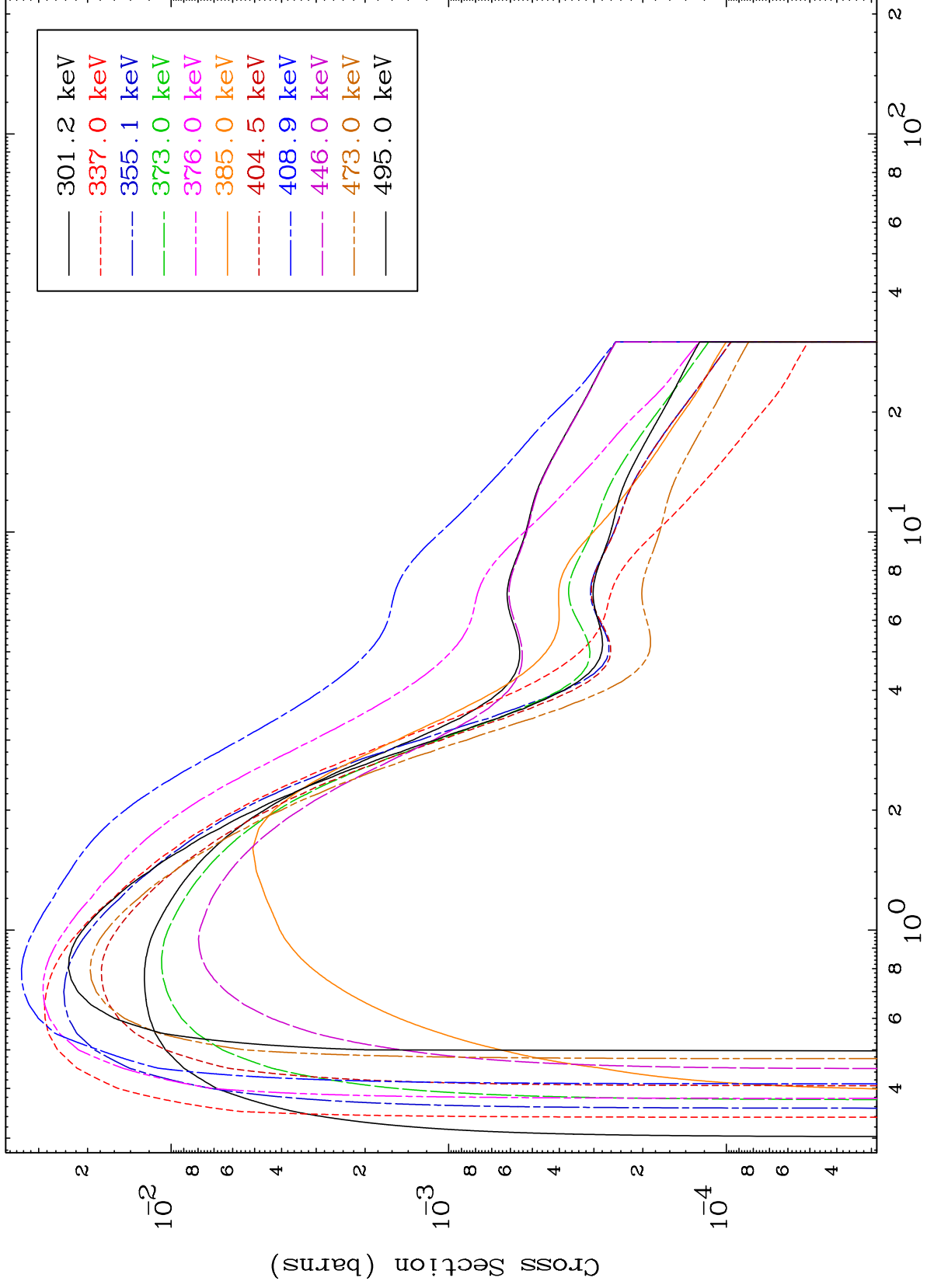
94-Pu-241



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(n,n') Level  
293 Kelvin Cross Sections

94-Pu-241



9

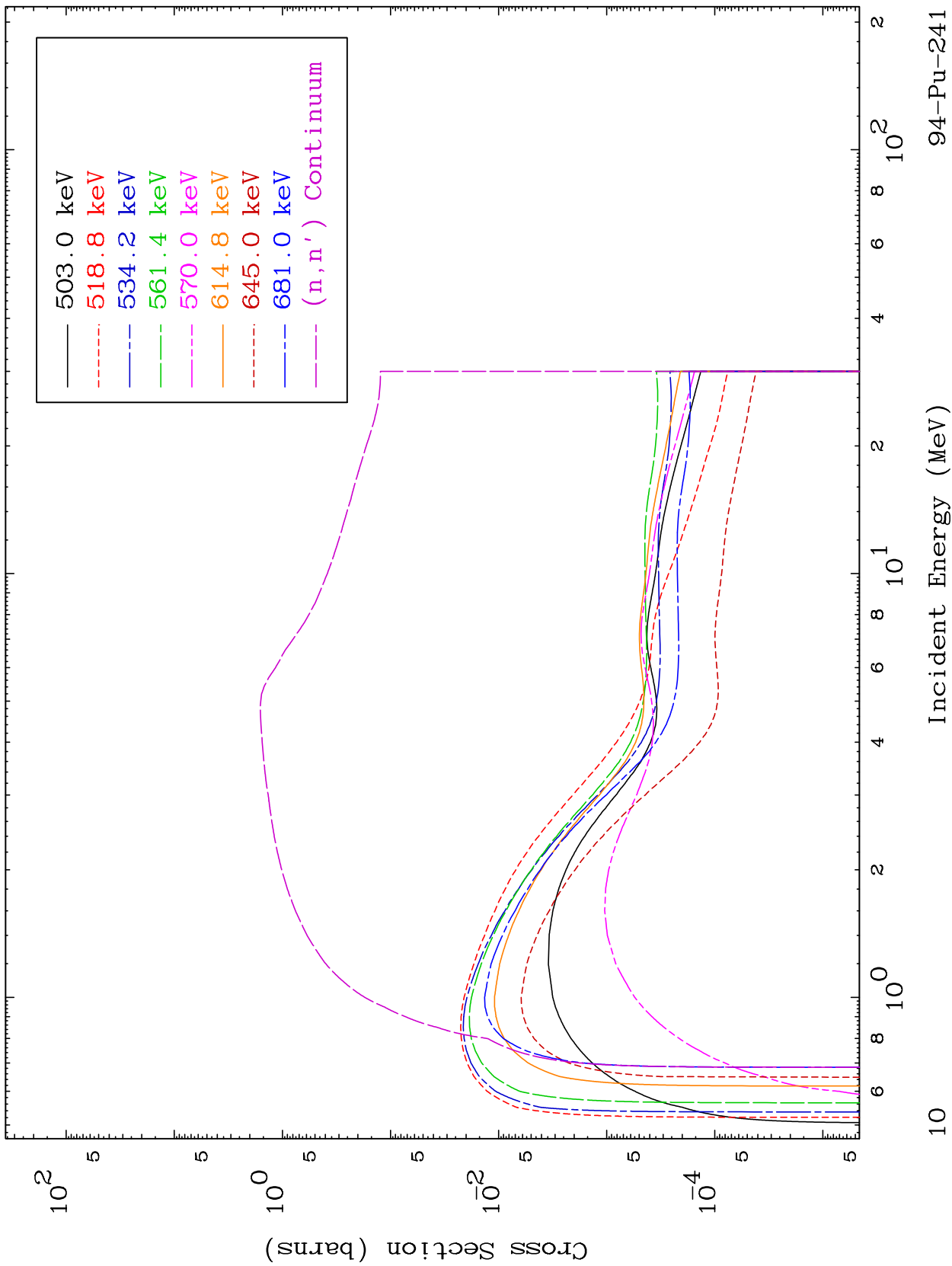
Incident Energy (MeV)

94-Pu-241

MAT 9443

(n,n') Level  
293 Kelvin Cross Sections

94-Pu-241



94-Pu-241

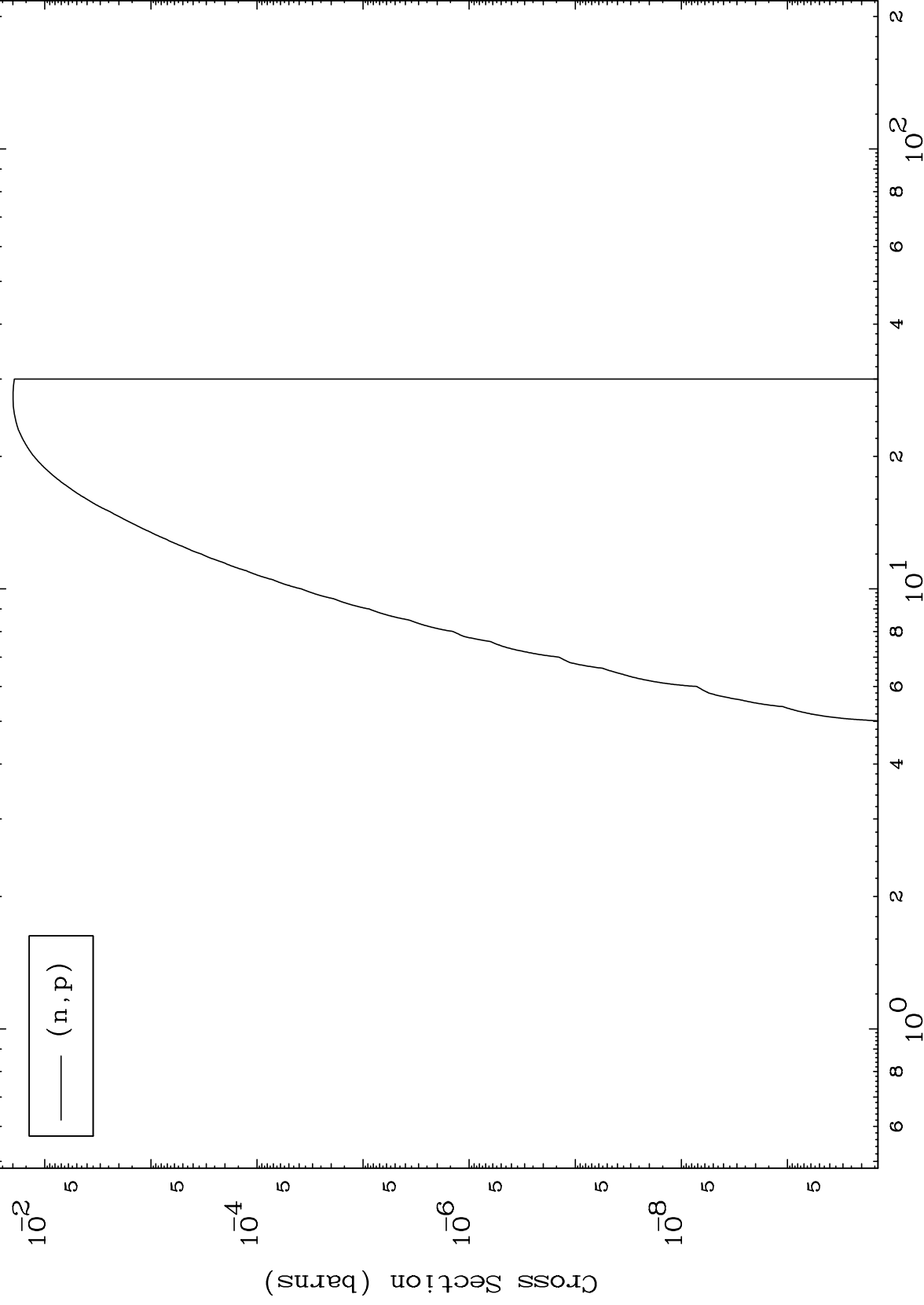
Incident Energy (MeV)

10

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(n,p) Levels  
293 Kelvin Cross Sections

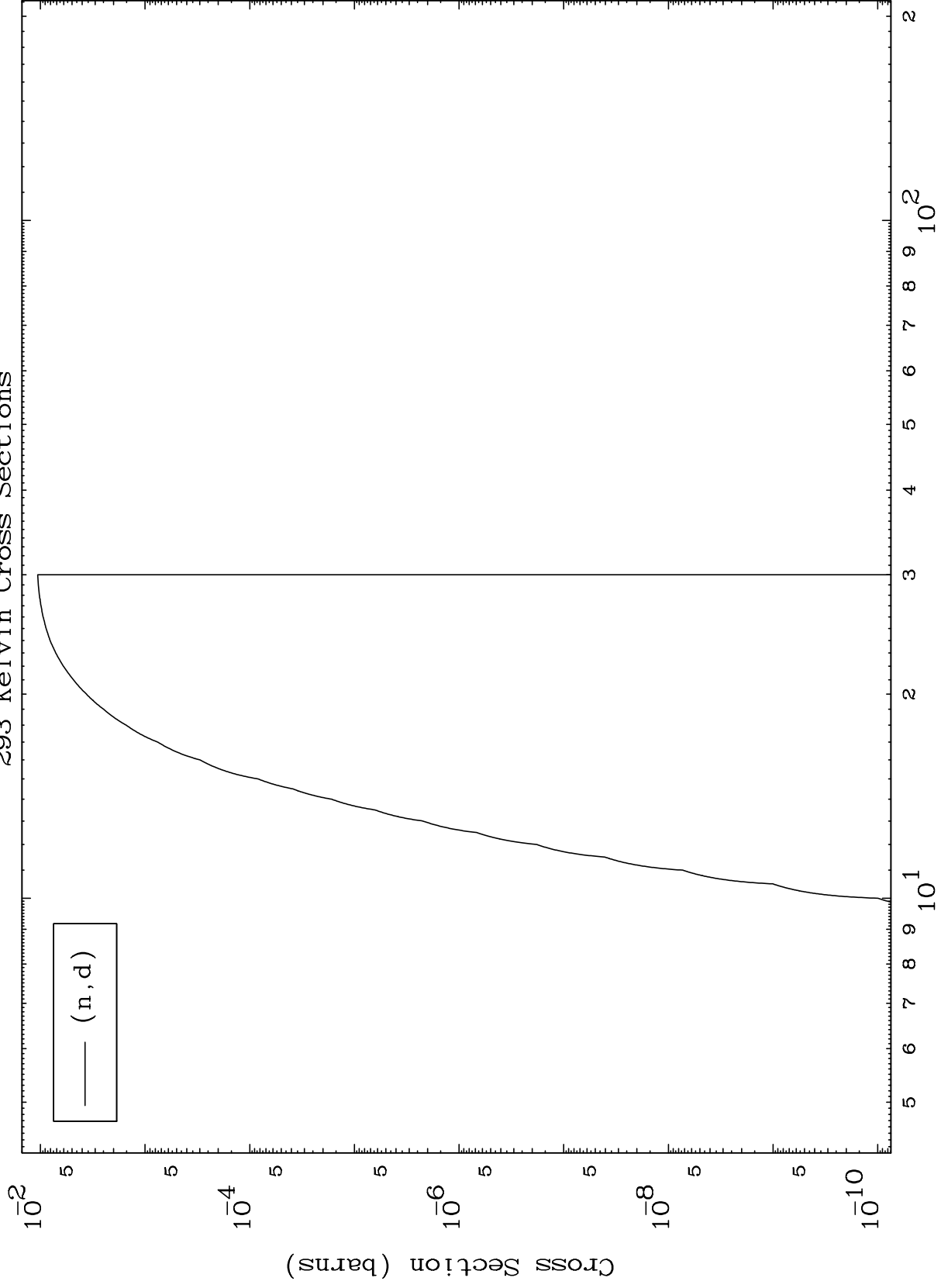
94-Pu-241



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(n,d) Levels  
293 Kelvin Cross Sections

94-Pu-241



12

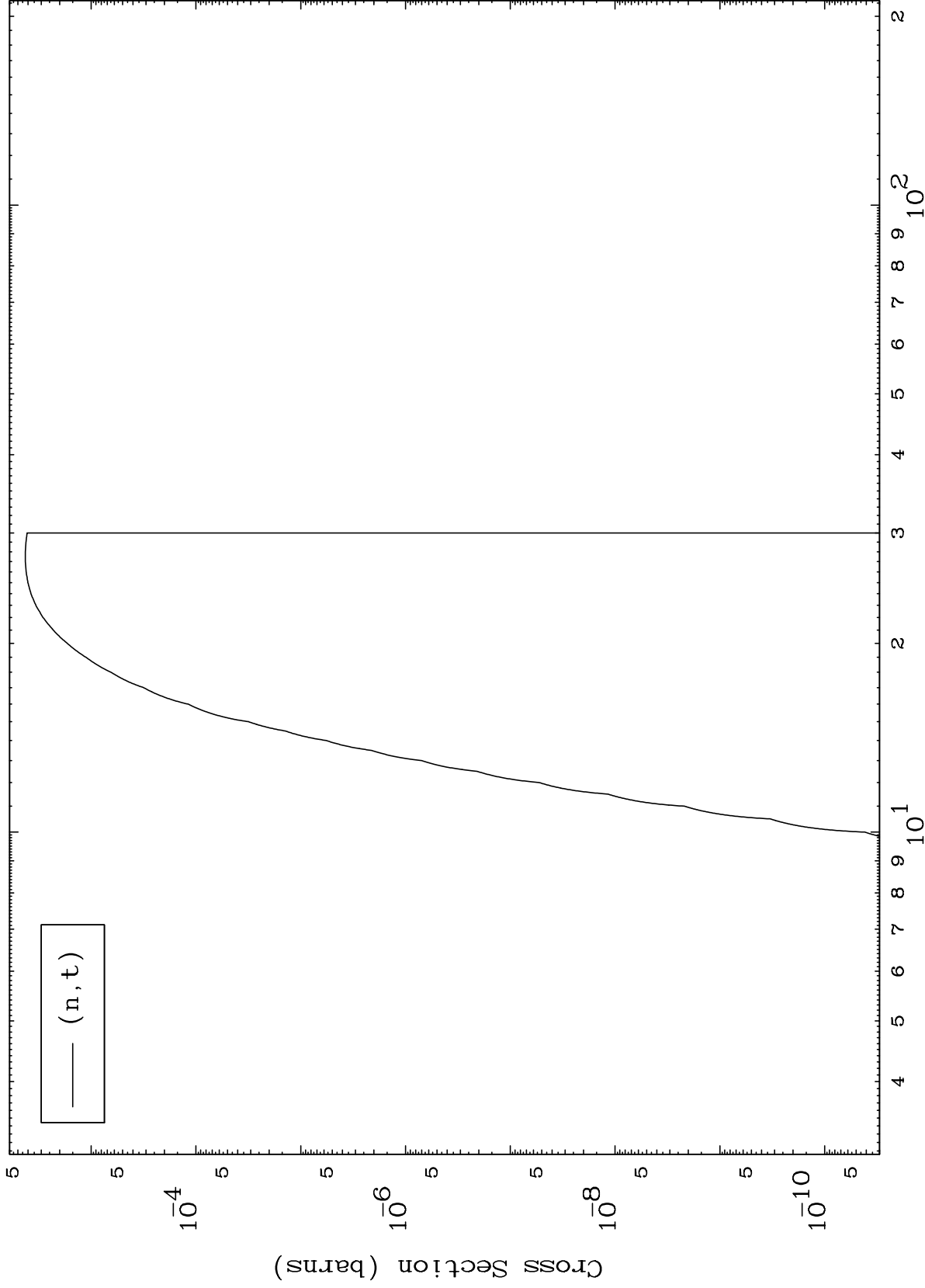
Incident Energy (MeV)

94-Pu-241

MAT 9443

(n,t) Levels  
293 Kelvin Cross Sections

94-Pu-241



13

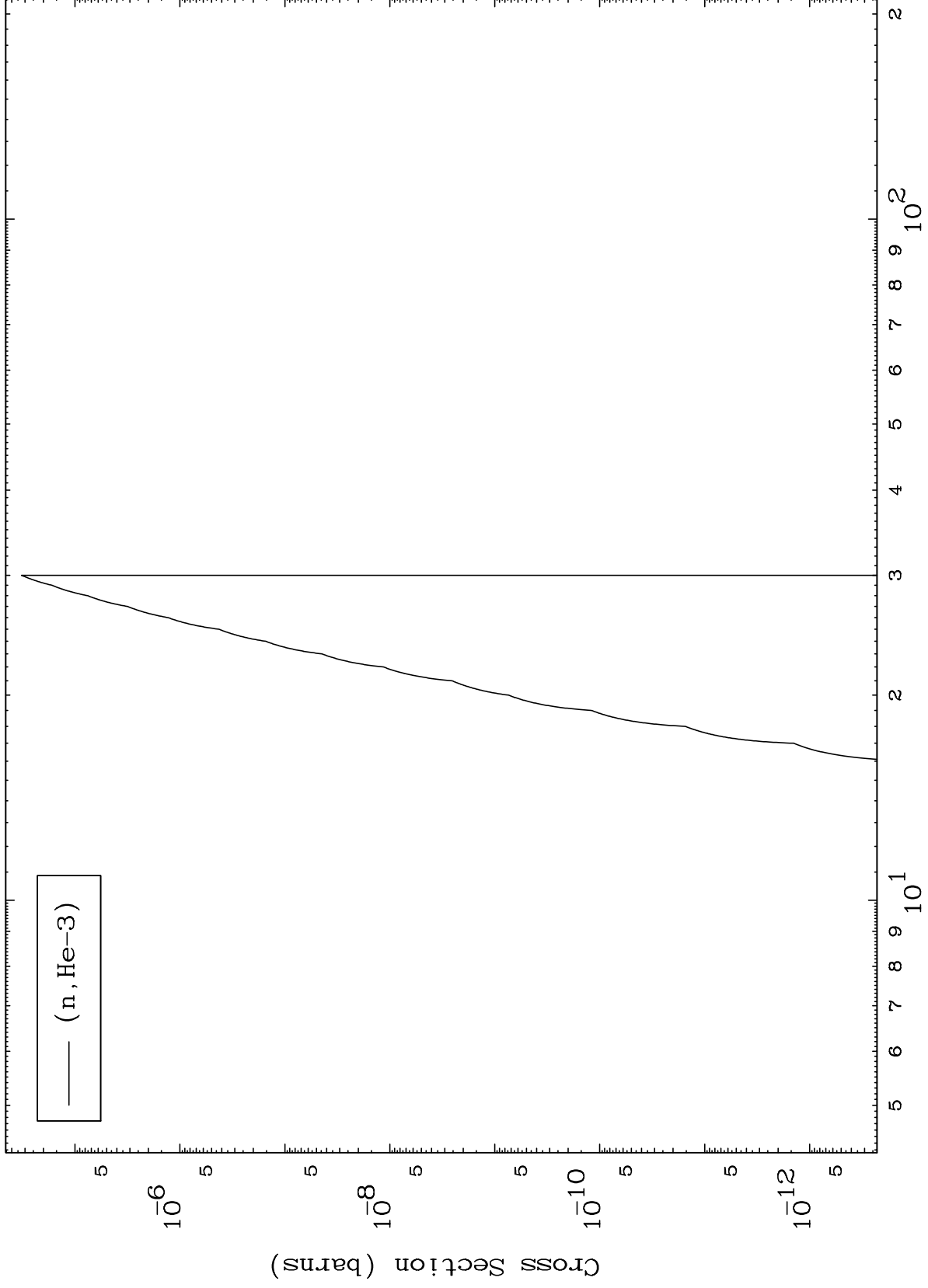
Incident Energy (MeV)

94-Pu-241

MAT 9443

(n,He3) Levels  
293 Kelvin Cross Sections

94-Pu-241



14

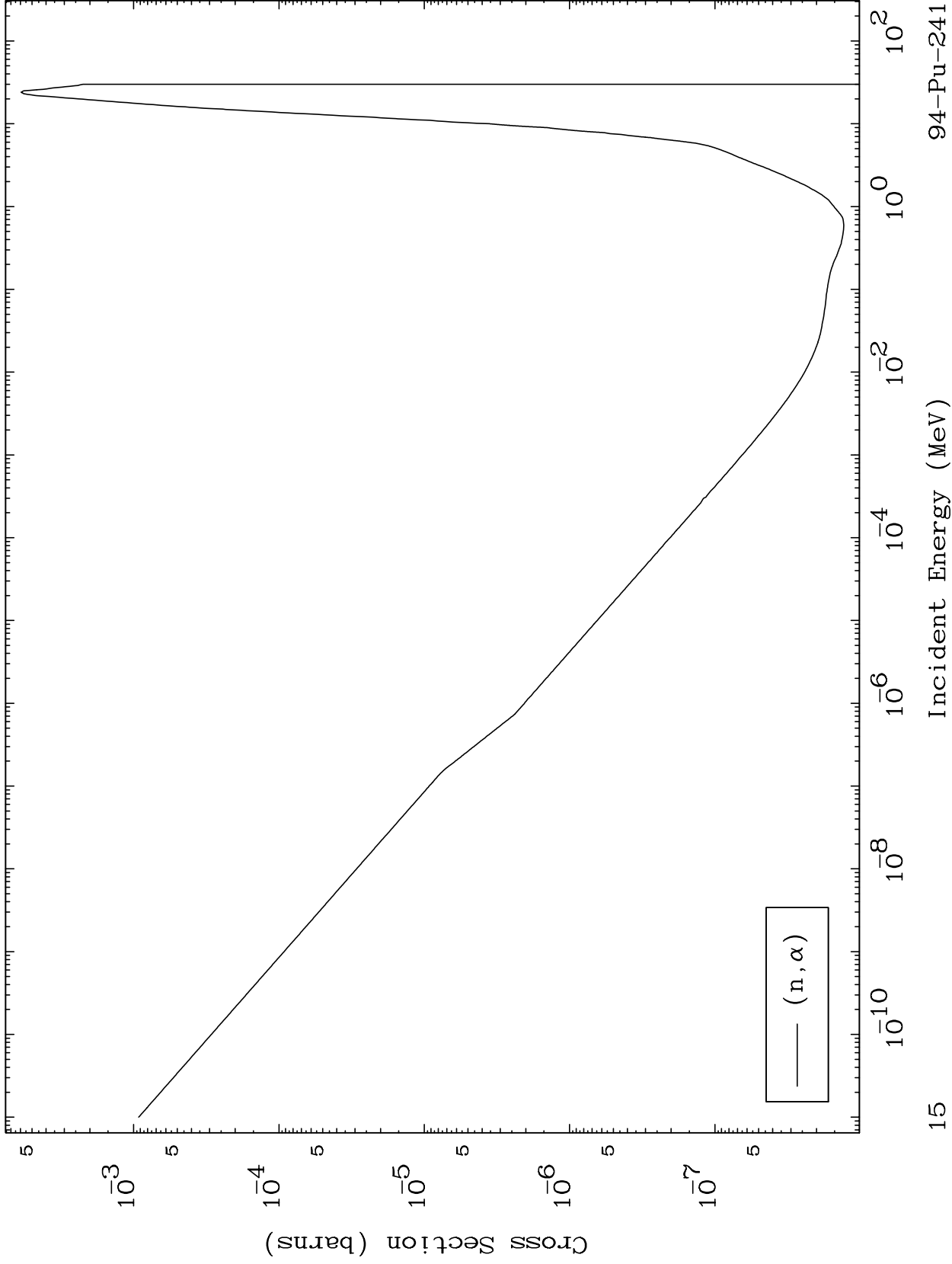
Incident Energy (MeV)

94-Pu-241

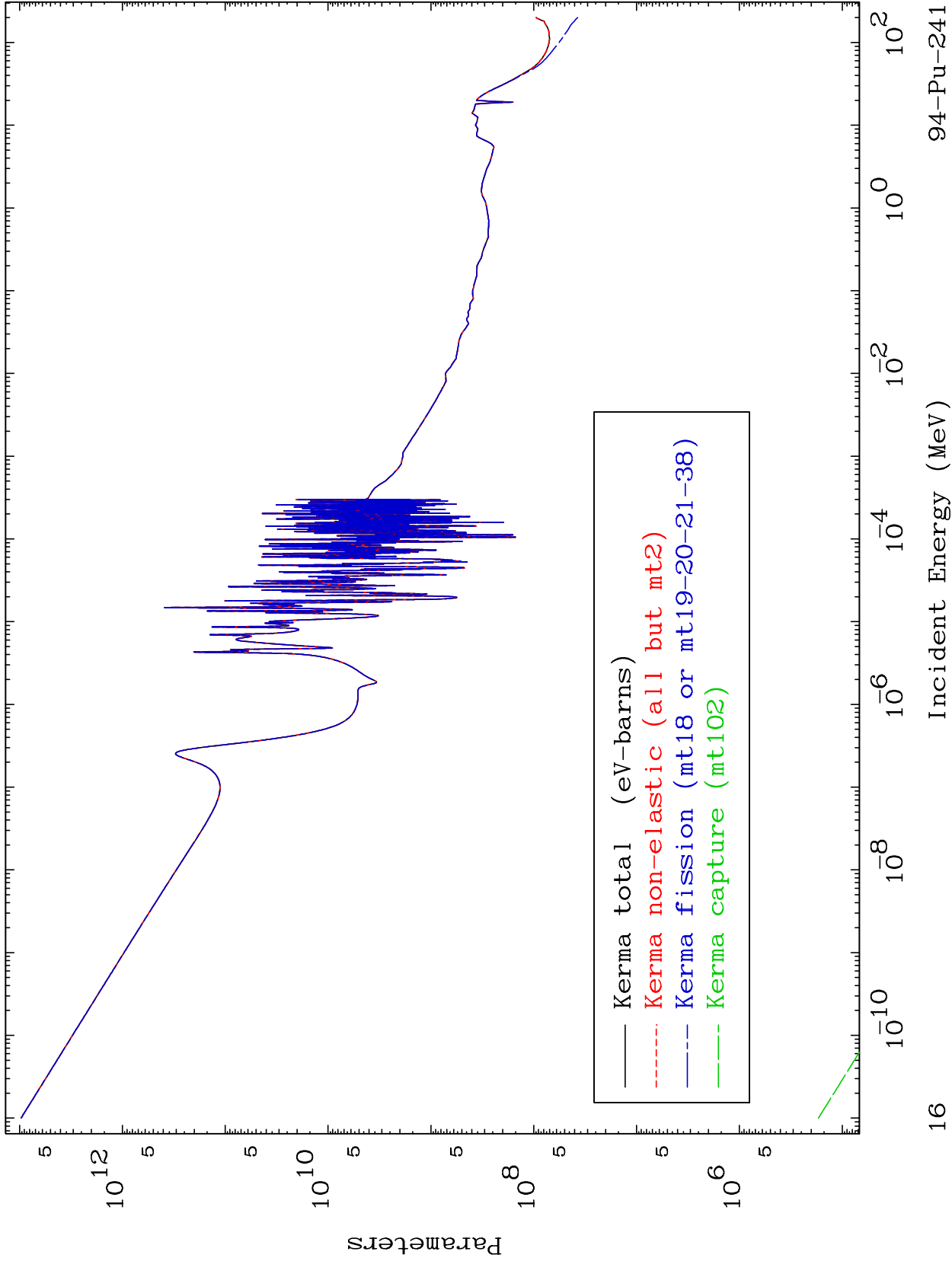
MAT 9443

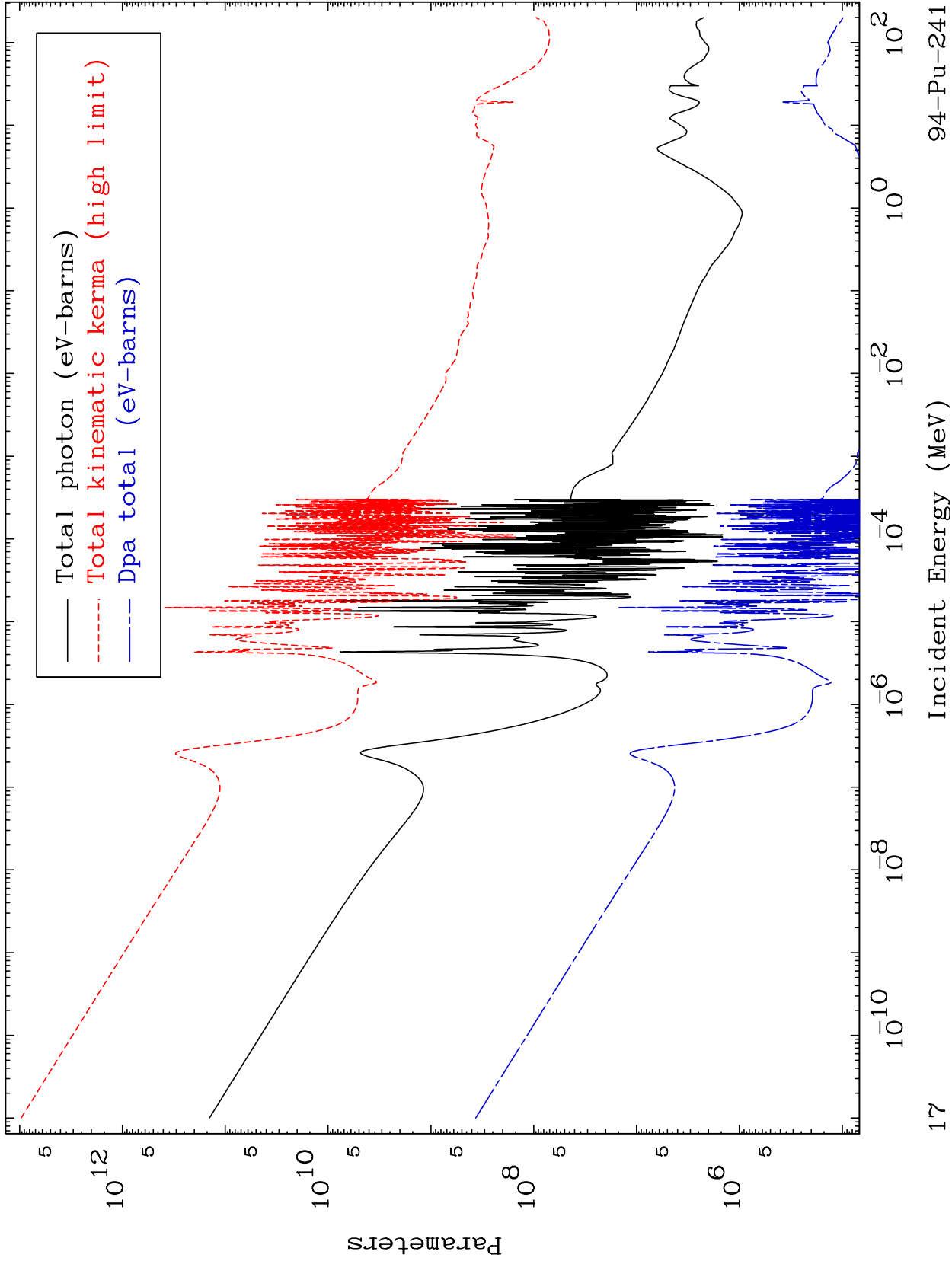
(n,  $\alpha$ ) Levels  
293 Kelvin Cross Sections

94-Pu-241





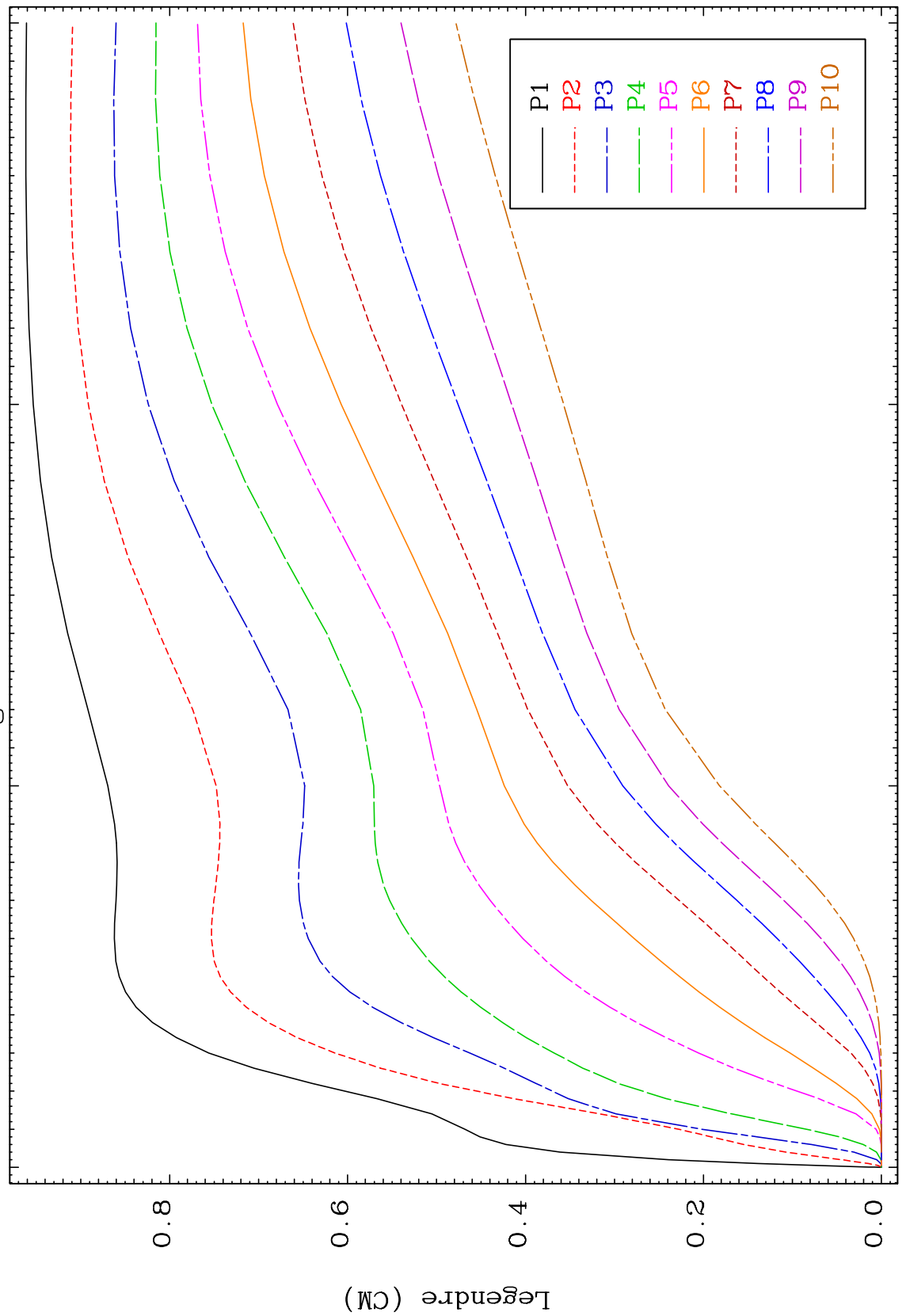




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94-Pu-241

Elastic Legendre Coefficients



94-Pu-241

Incident Energy (MeV)

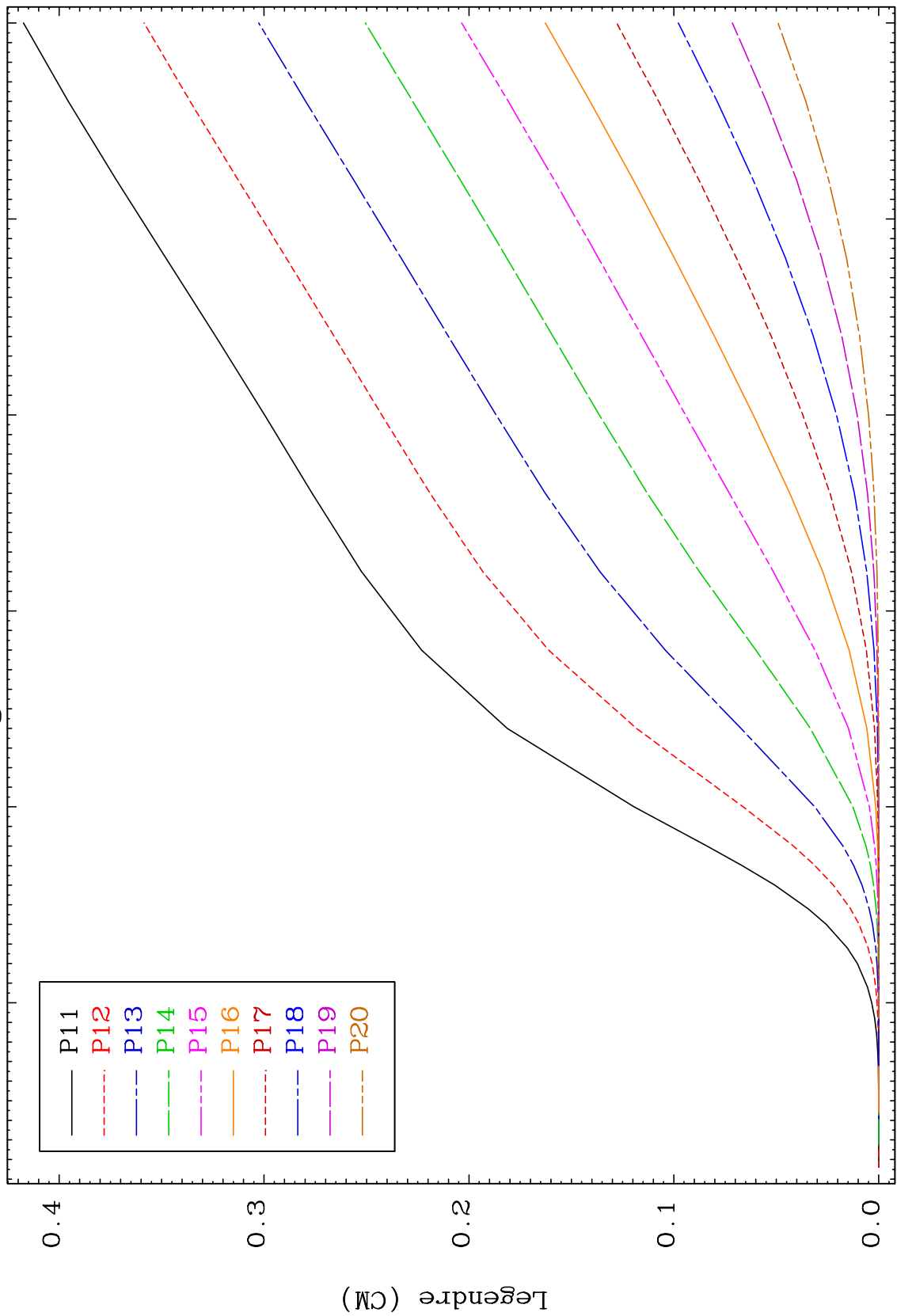
18

30

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Elastic Legendre Coefficients

94-Pu-241



19

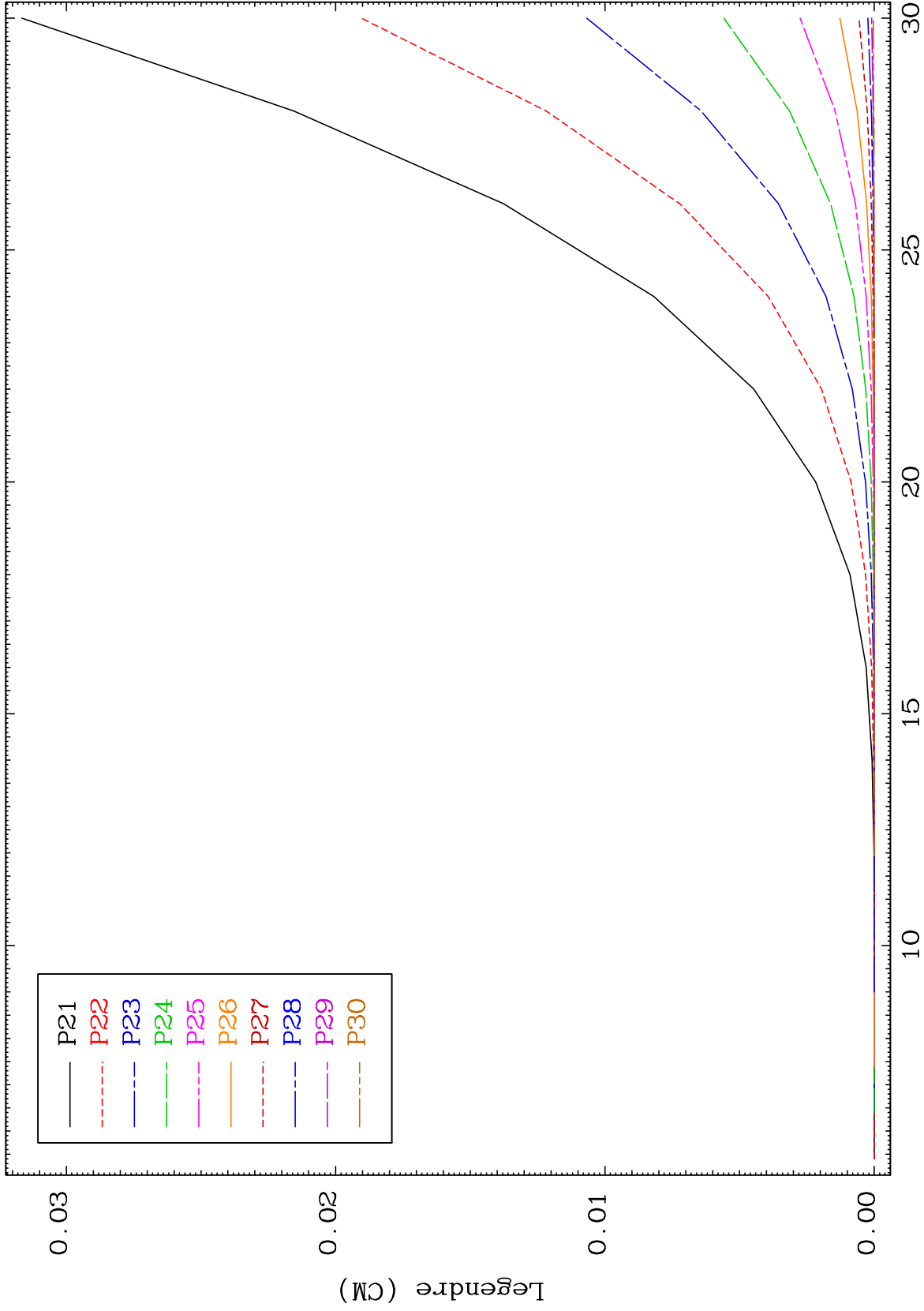
Incident Energy (MeV)

94-Pu-241

MAT 9443

Elastic Legendre Coefficients

94-Pu-241



20

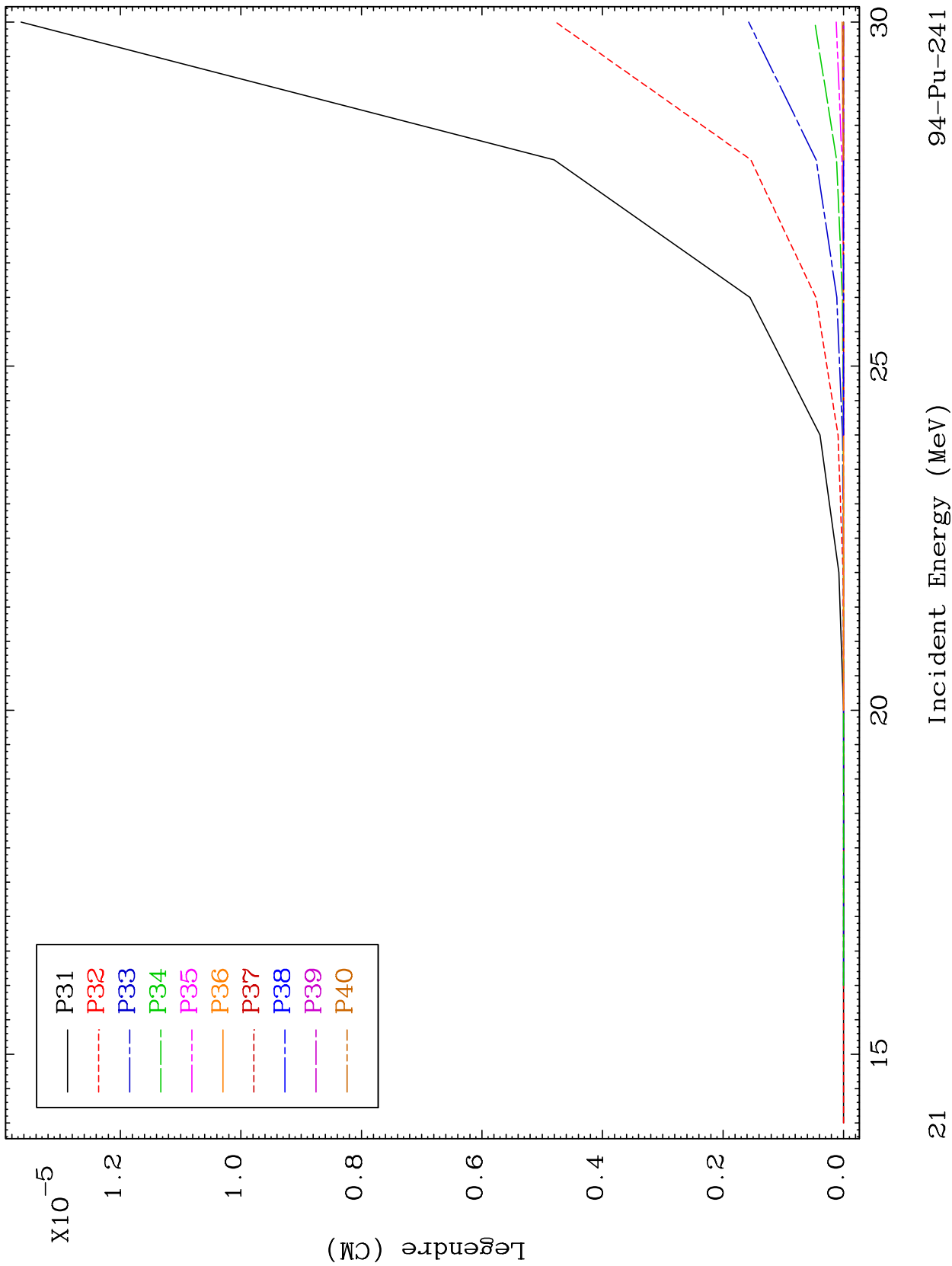
Incident Energy (MeV)

94-Pu-241

MAT 9443

Elastic Legendre Coefficients

94-Pu-241



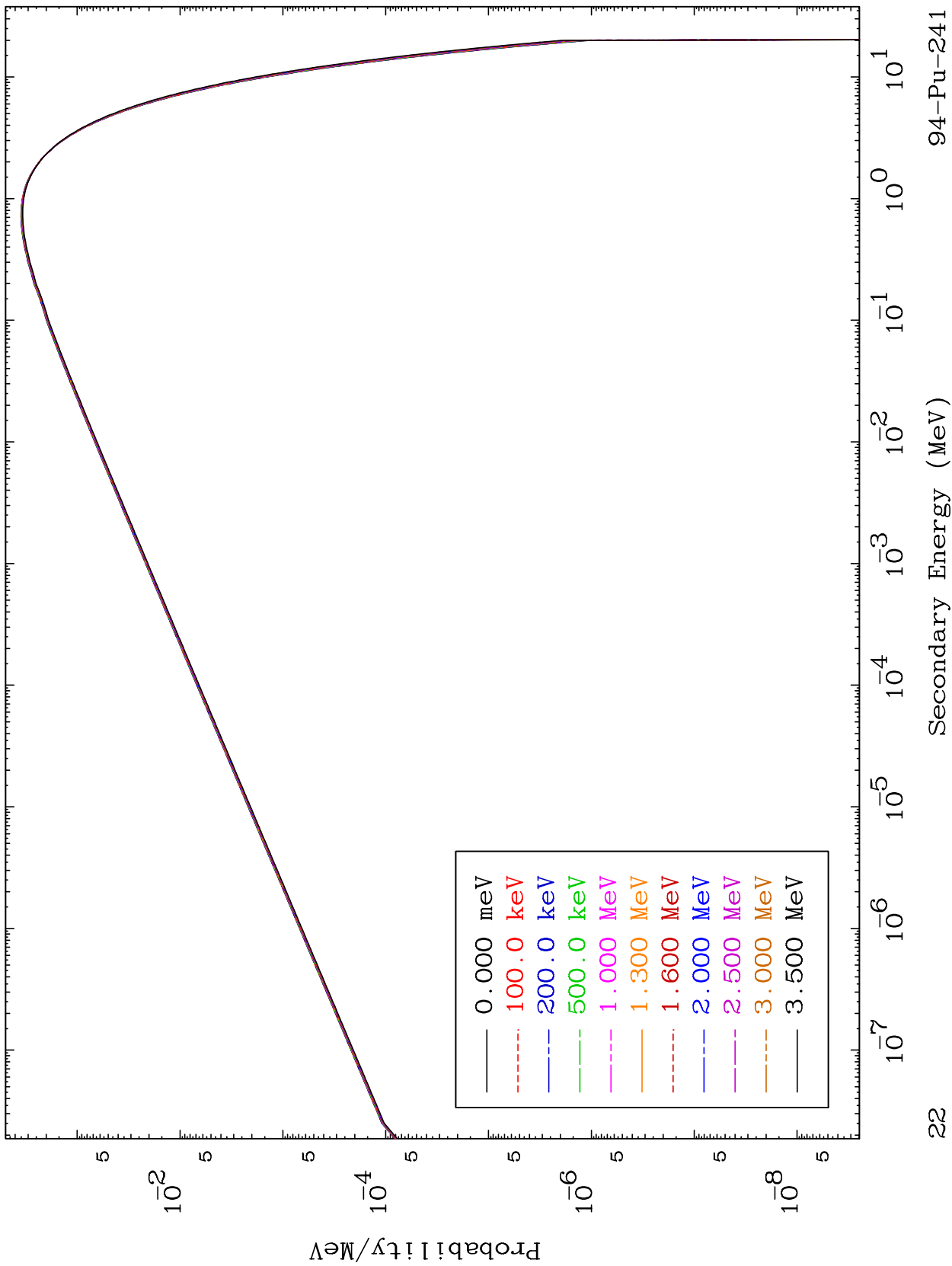
21

94-Pu-241

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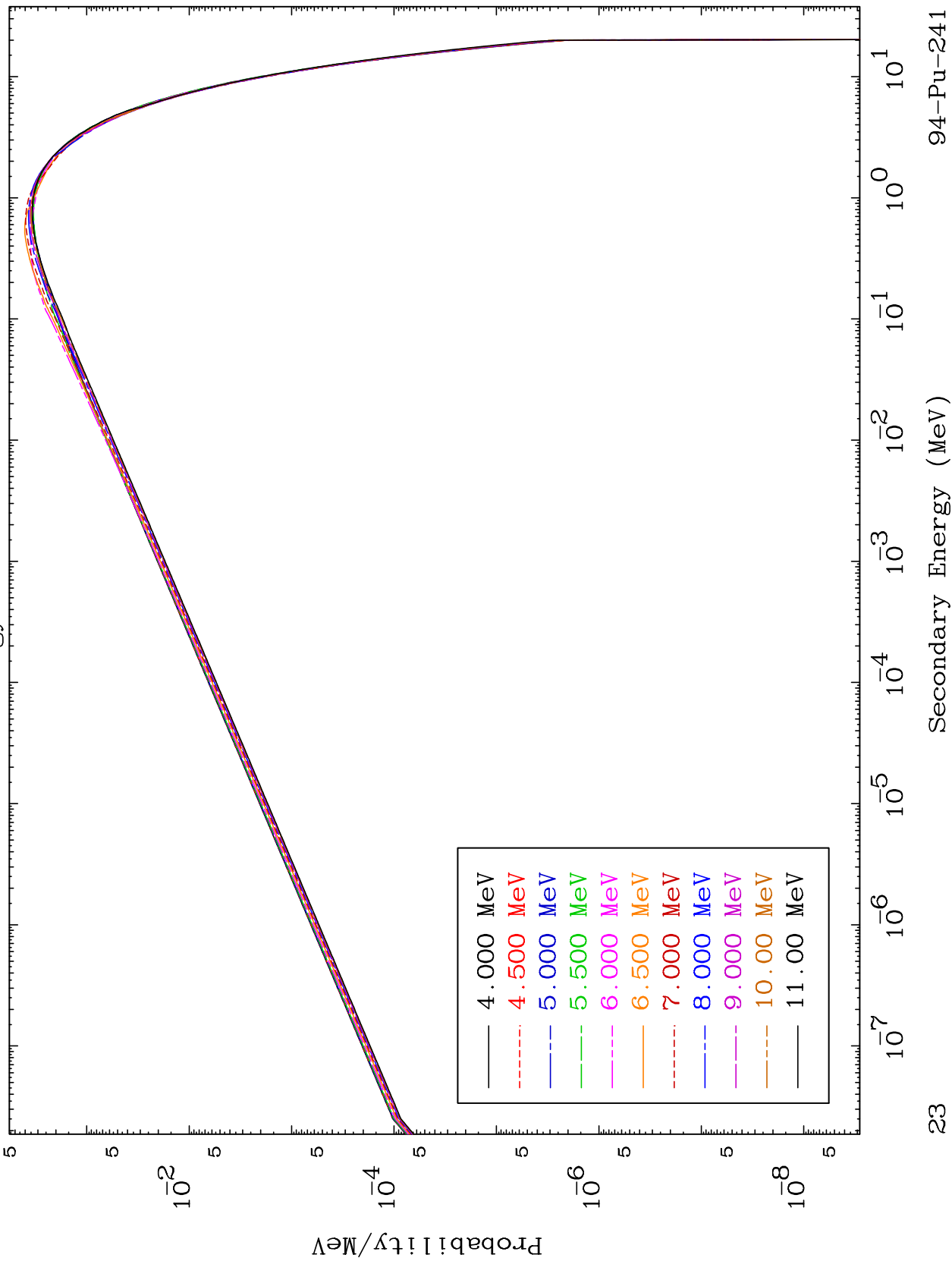
### Fission Energy Distribution

94-Pu-241



94-Pu-241

Fission  
Energy Distribution

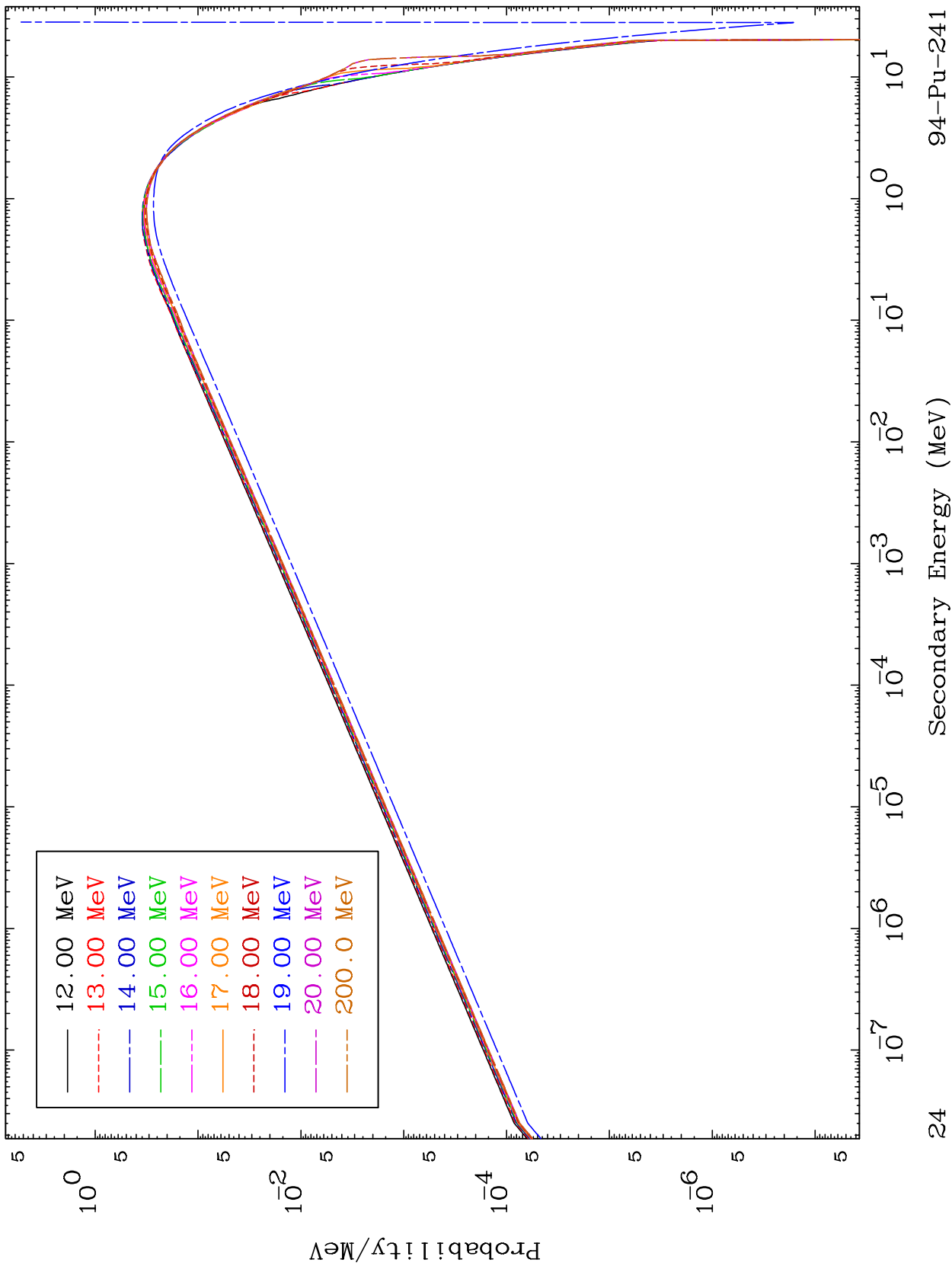




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### Fission Energy Distribution

94-Pu-241



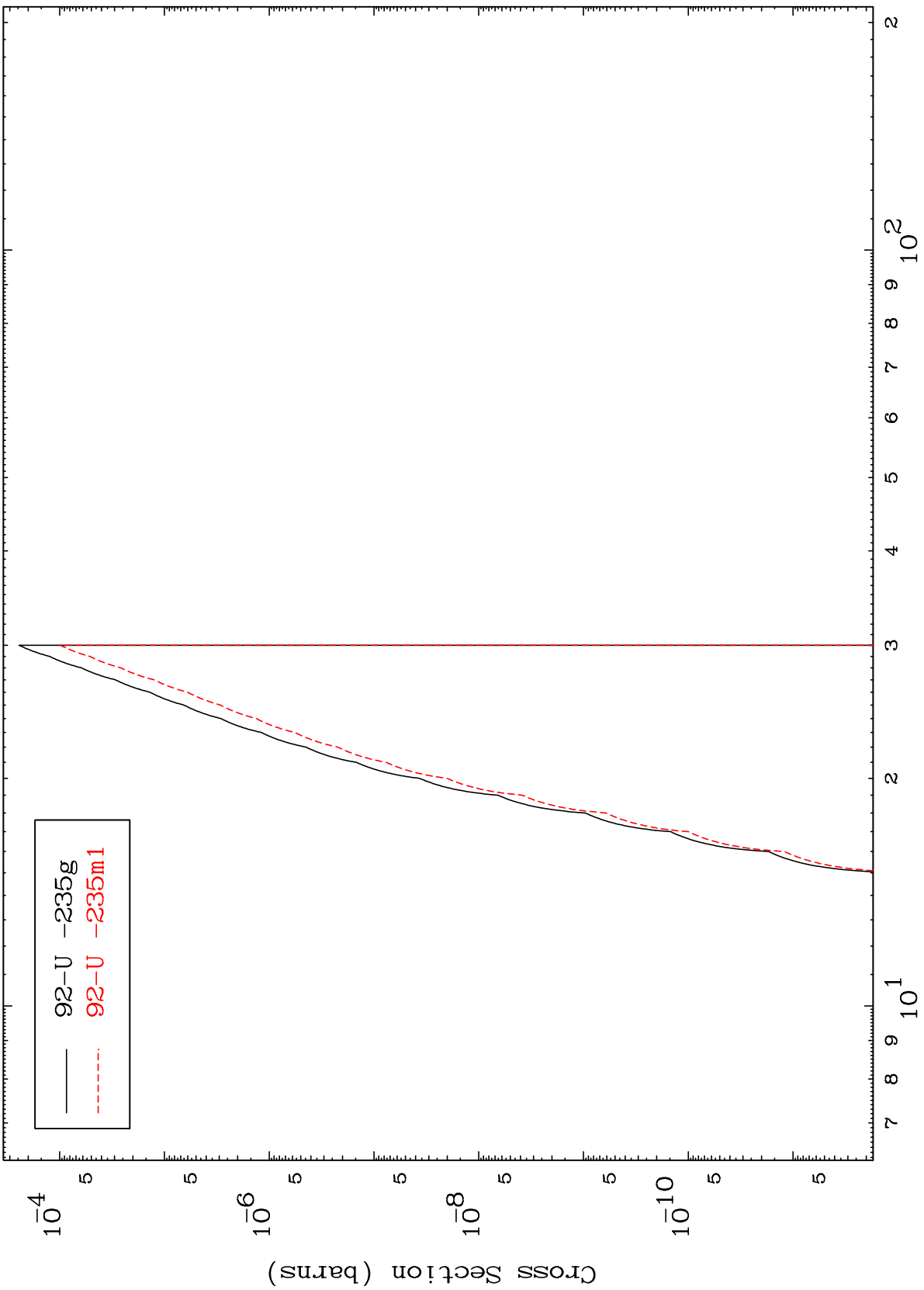
94-Pu-241

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(n,3n)  $\alpha$

94-Pu-241

Radionuclide Production Cross Section



92-U -235g  
92-U -235m1

25

Incident Energy (MeV)

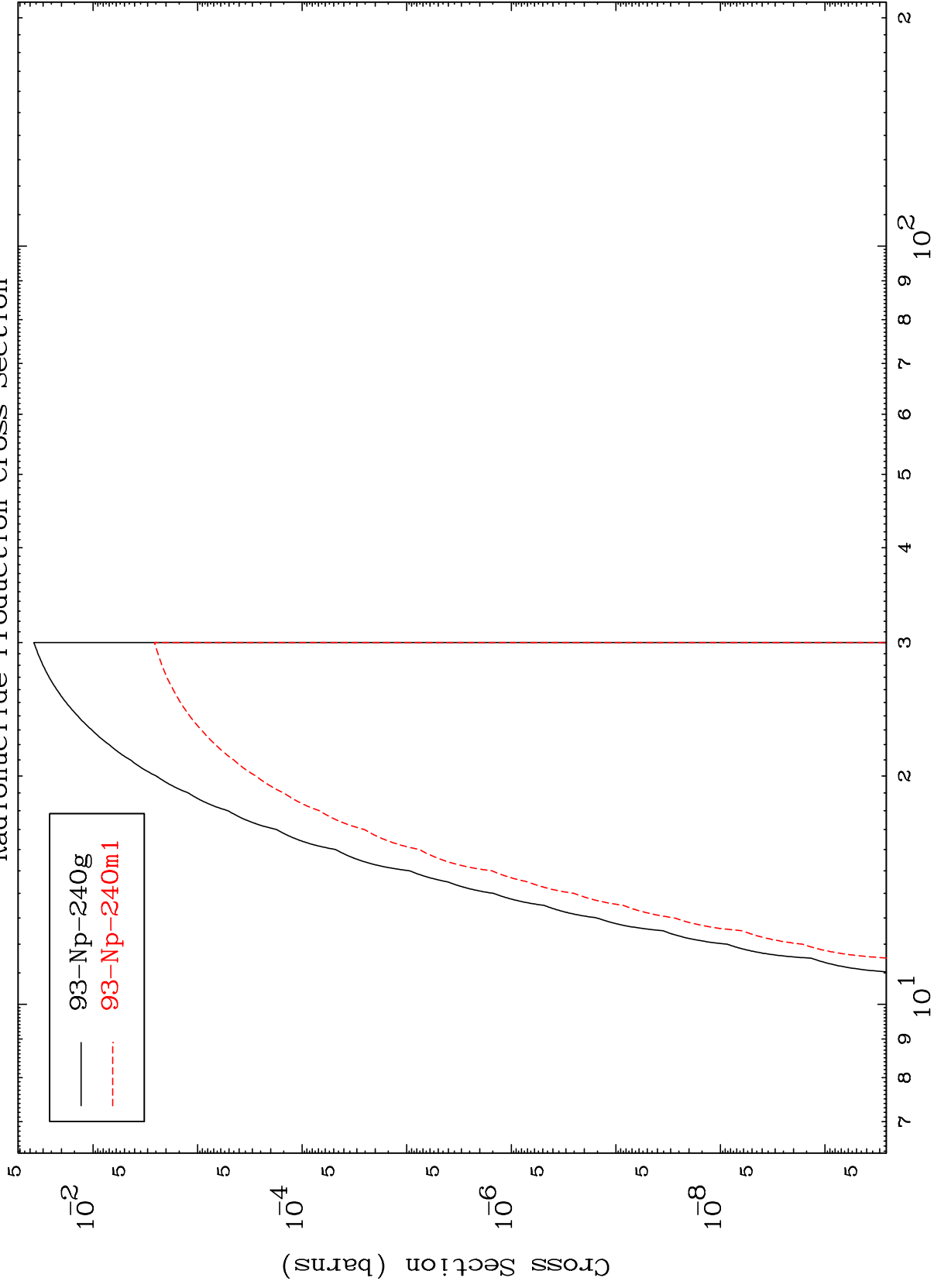
94-Pu-241

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(n,n') p

94-Pu-241

Radionuclide Production Cross Section



26

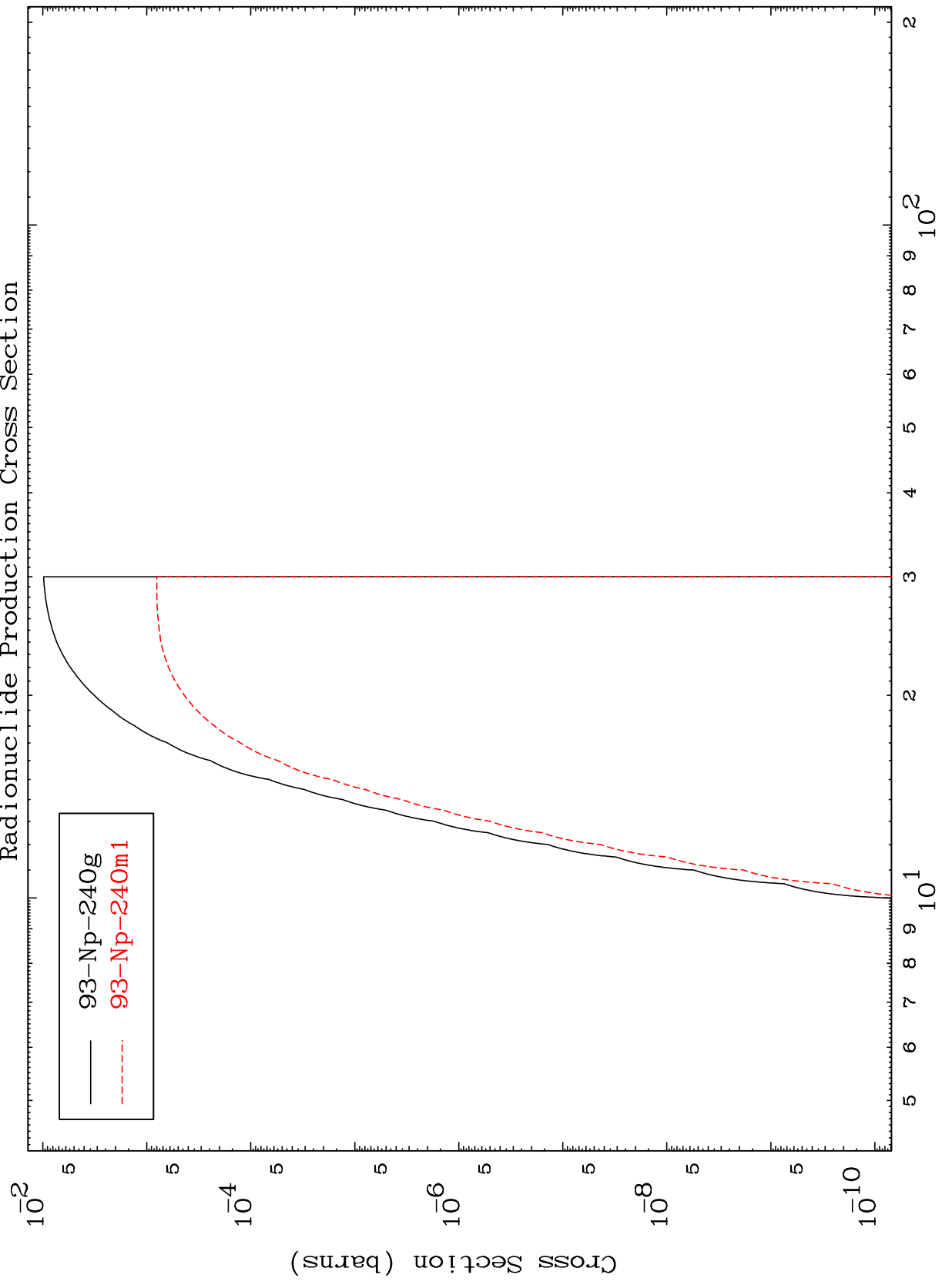
Incident Energy (MeV)

94-Pu-241

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94-Pu-241

Radionuclide Production Cross Section (n,d)



Incident Energy (MeV)

94-Pu-241

27