

Program EVALPLOT
(Version 2015-2)

by

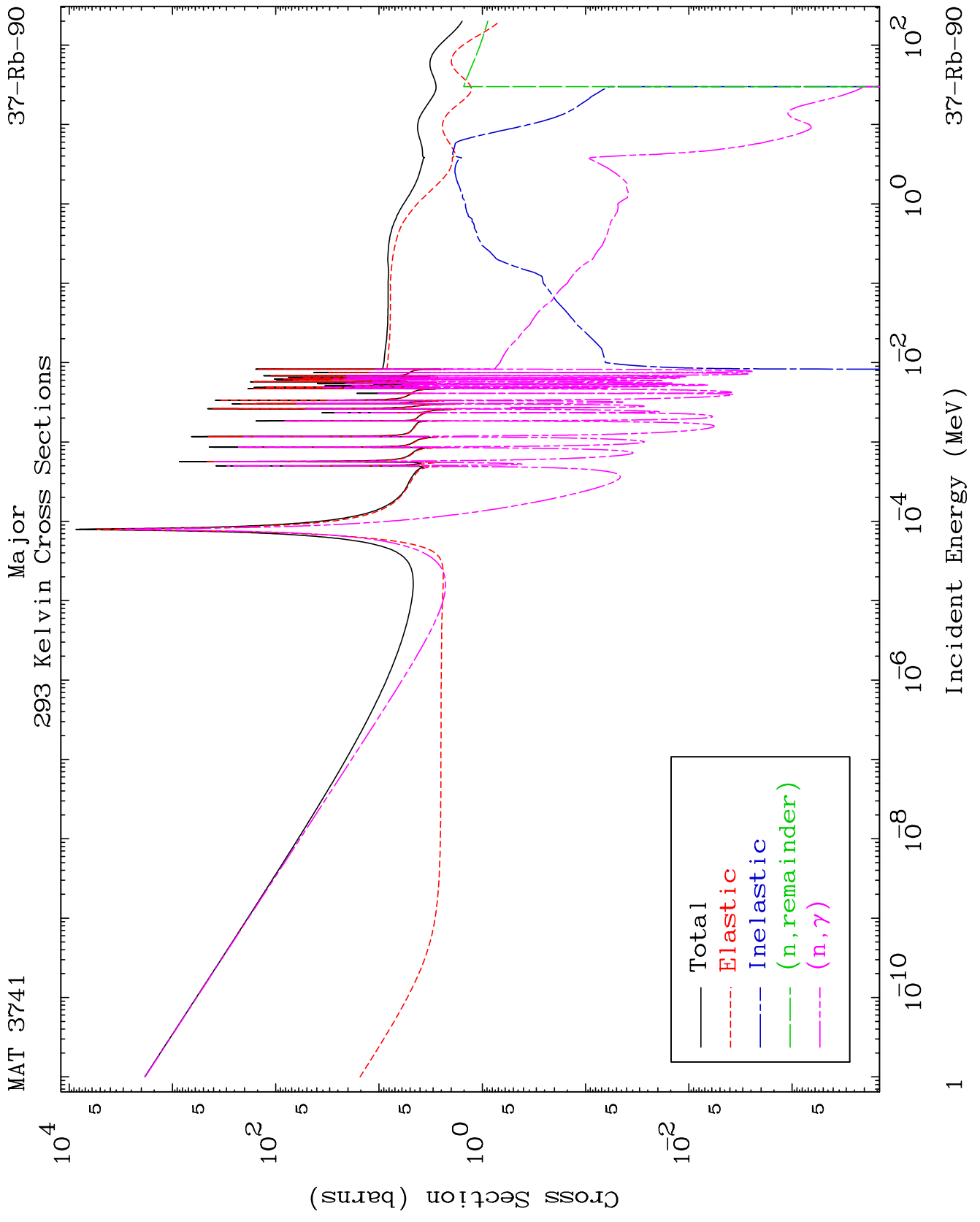
Dermott E. Cullen
(Present Contact Information)

Dermott E. Cullen
1466 Hudson Way
Livermore, CA 94550
U.S.A.

Tele: 925-443-1911

E.Mail:redcullen1@comcast.net
Web:home.comcast.net/~redcullen1

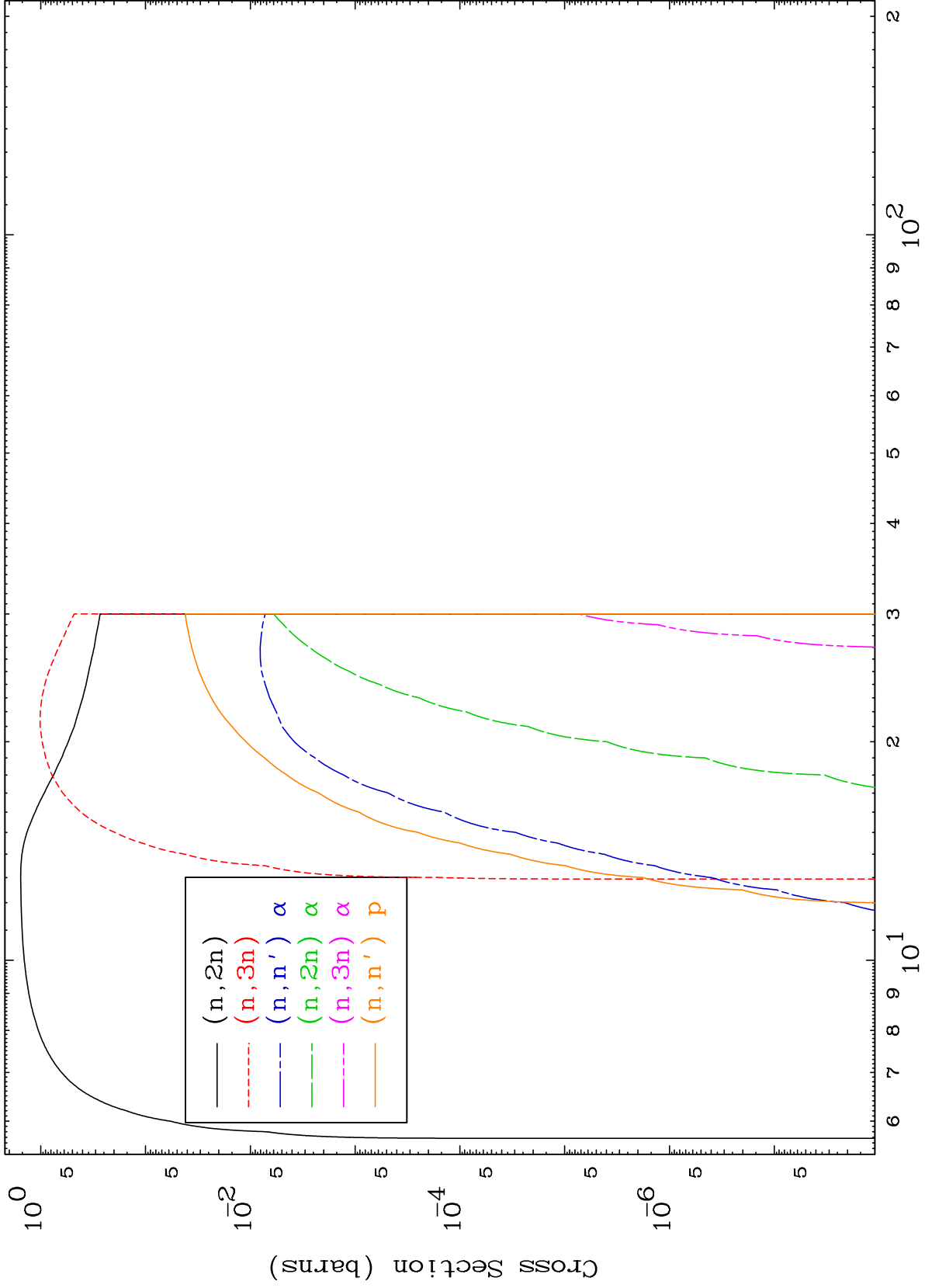
Press Mouse Button to Start



MAT 3741

Neutron Production
293 Kelvin Cross Sections

37-Rb-90



2

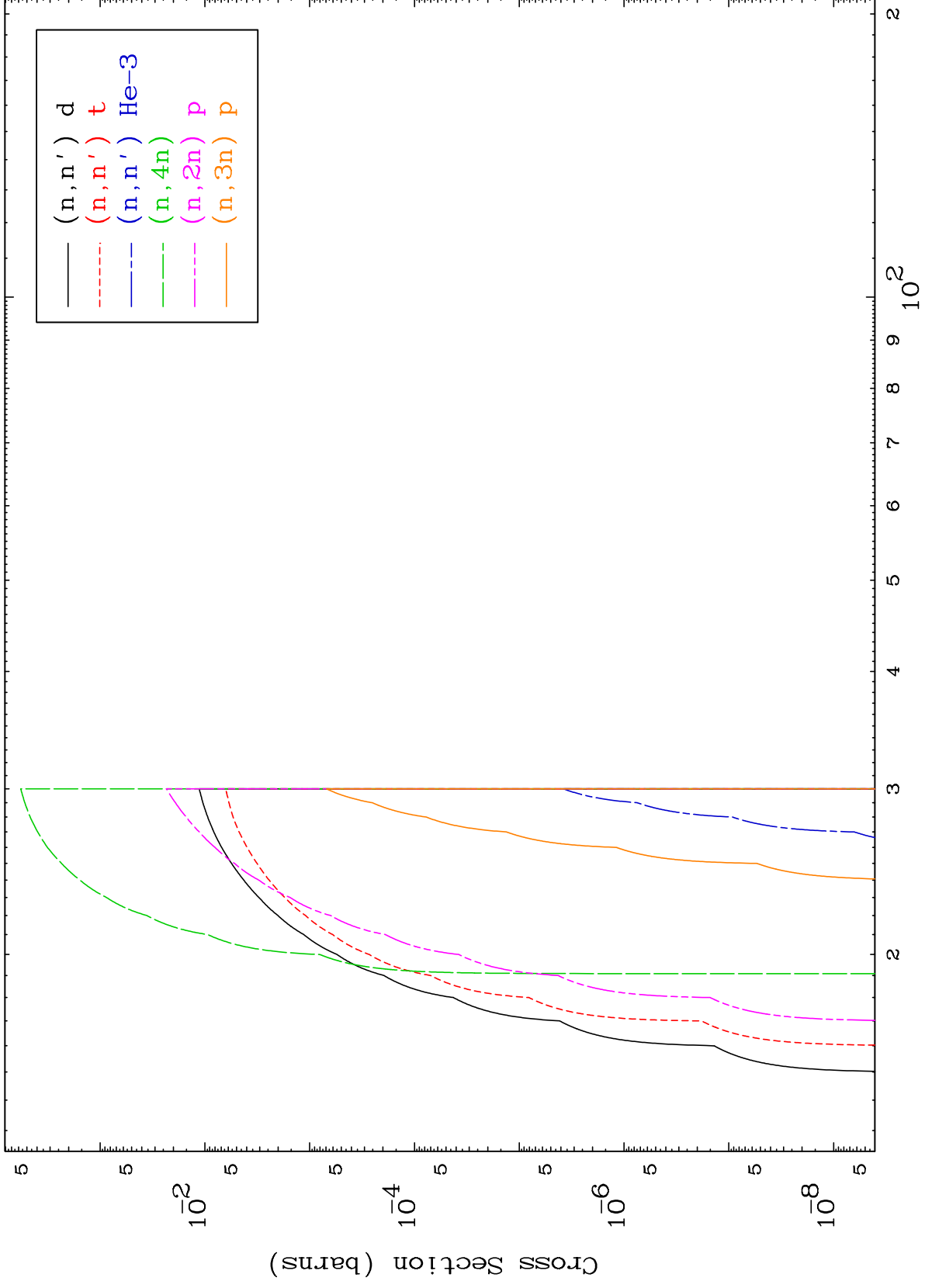
Incident Energy (MeV)

37-Rb-90

MAT 3741

Neutron Production
293 Kelvin Cross Sections

37-Rb-90



3

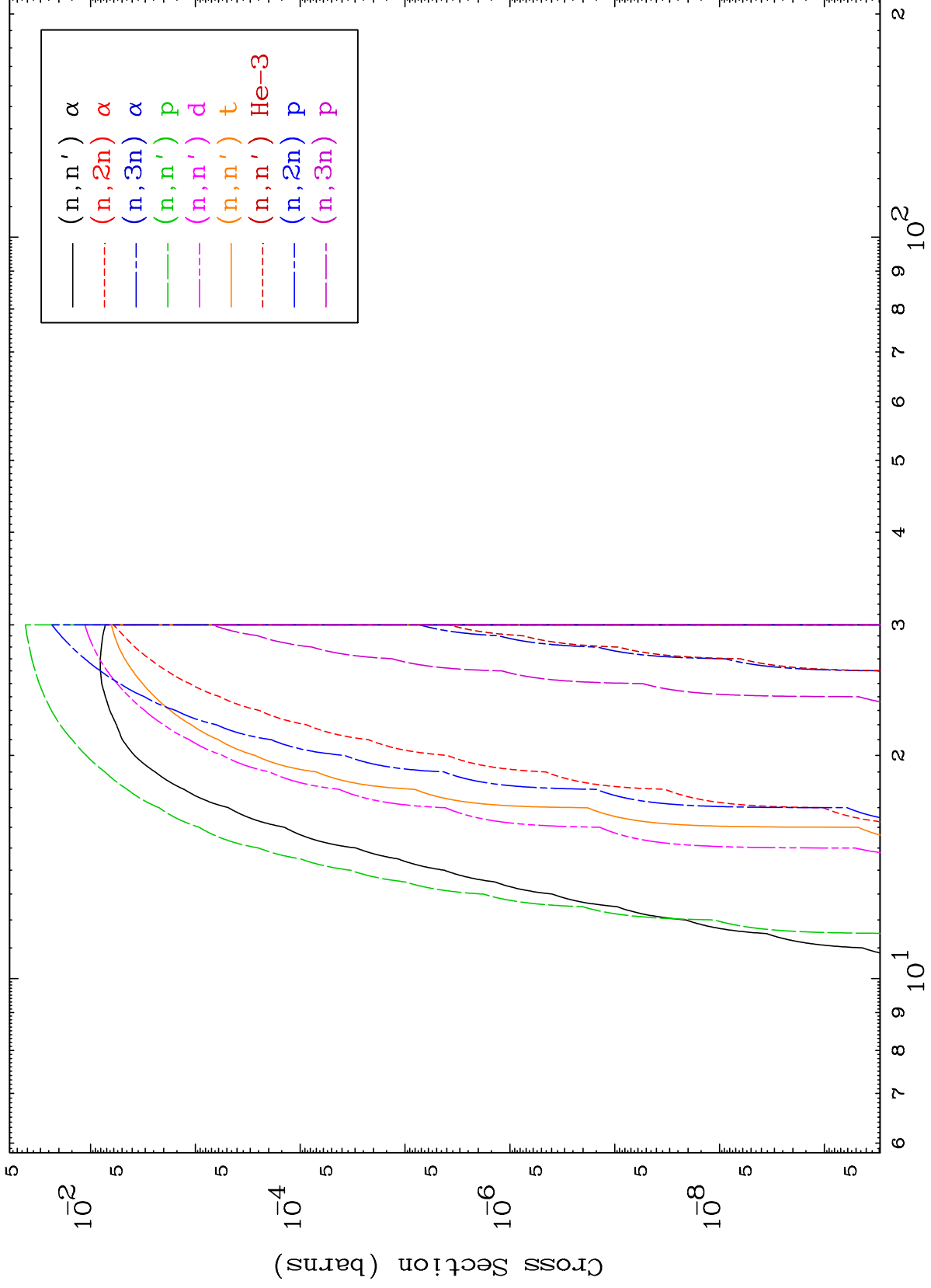
Incident Energy (MeV)

37-Rb-90

MAT 3741

Charged Particle
293 Kelvin Cross Sections

37-Rb-90



4

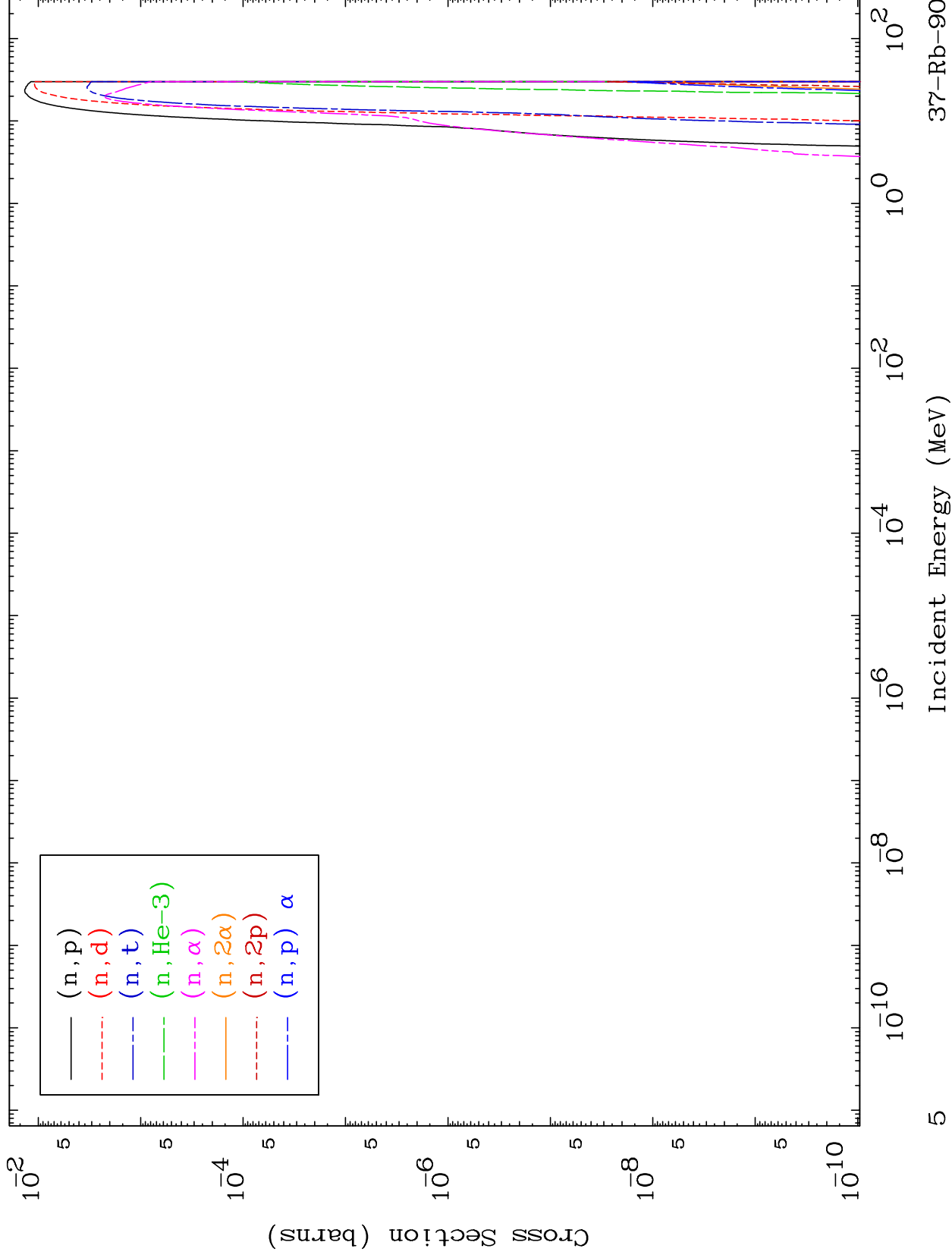
Incident Energy (MeV)

37-Rb-90

MAT 3741

Charged Particle
293 Kelvin Cross Sections

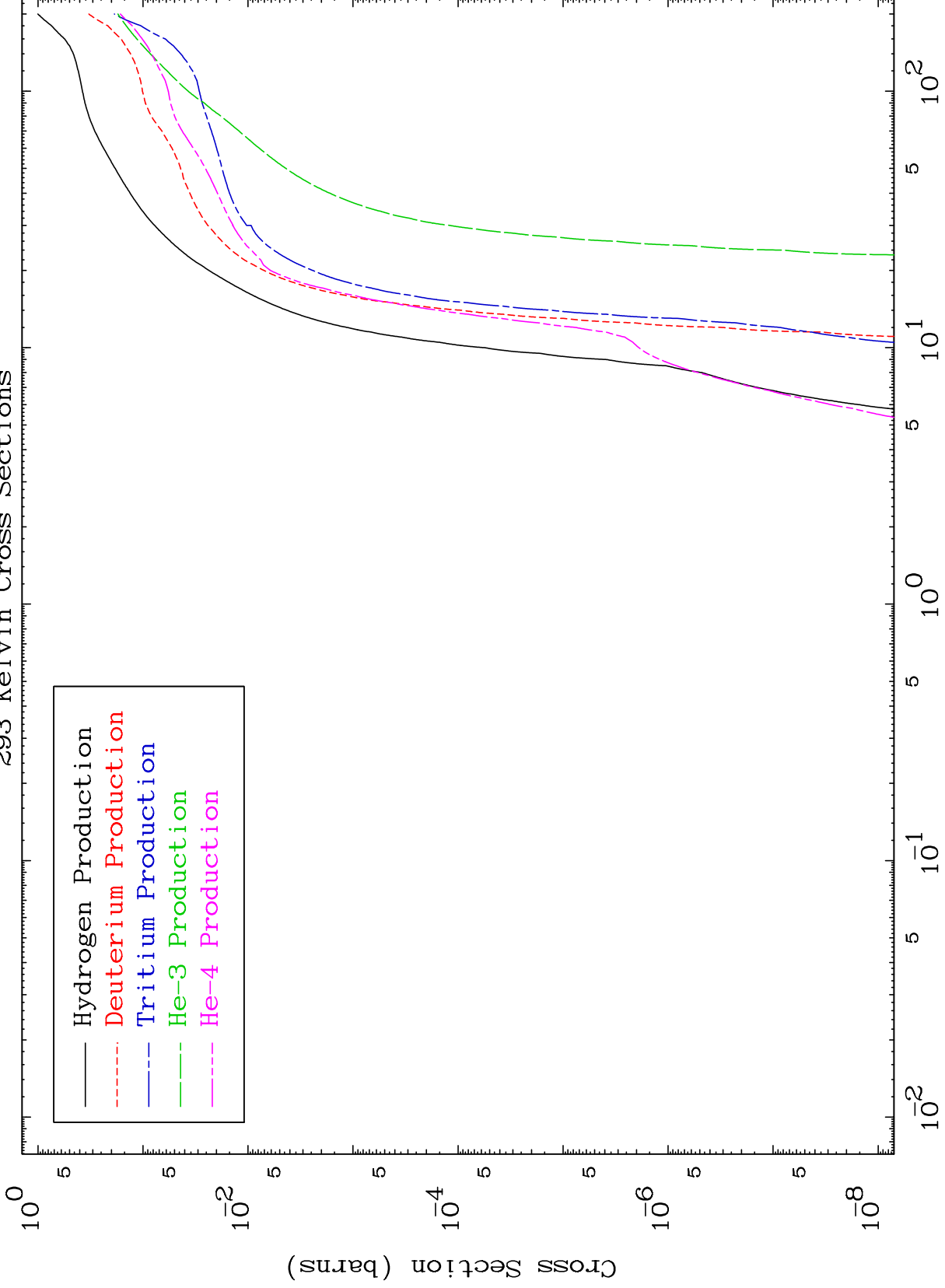
37-Rb-90



MAT 3741

Particle Production
293 Kelvin Cross Sections

37-Rb-90



37-Rb-90

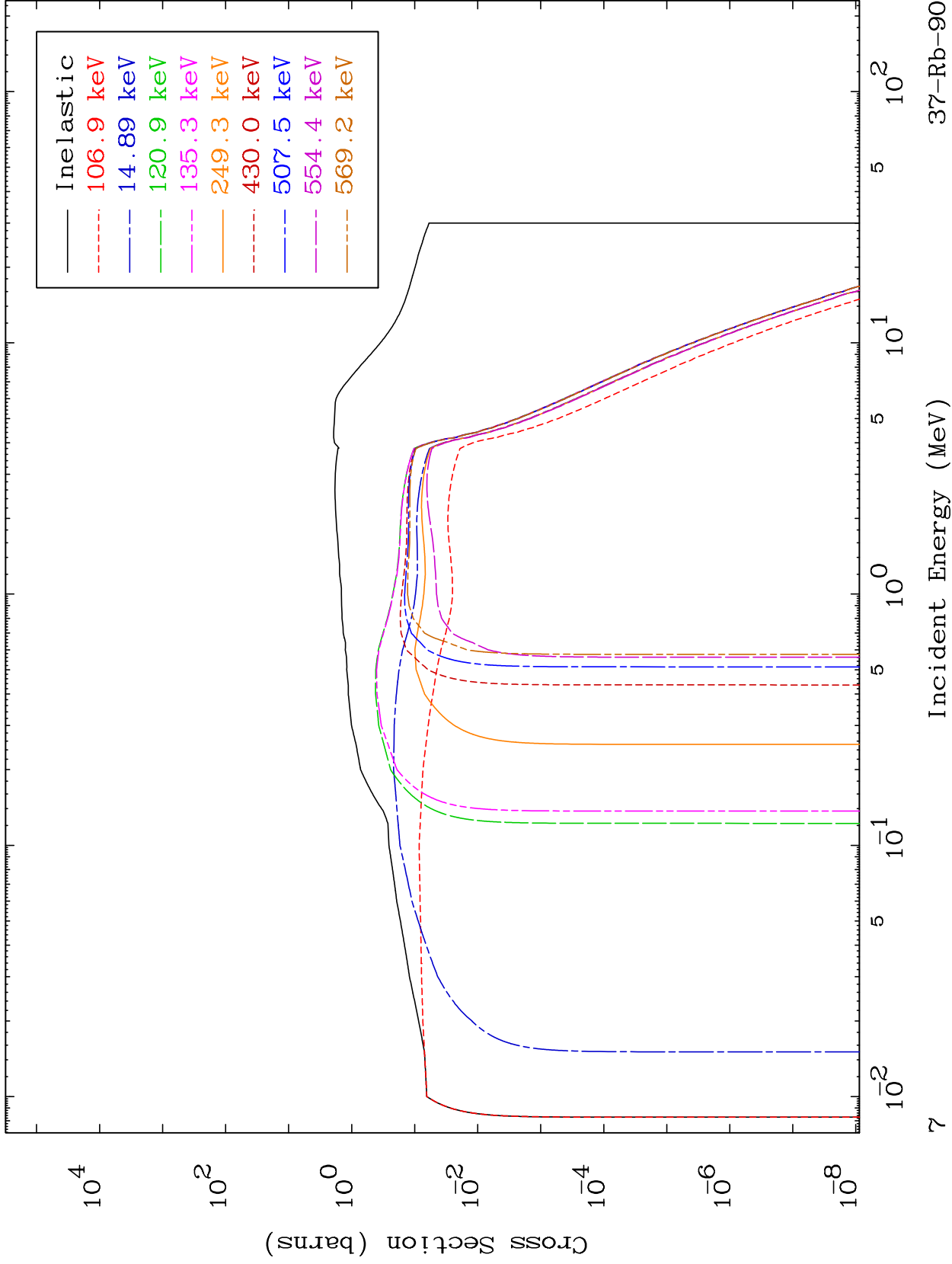
Incident Energy (MeV)

6

MAT 3741

(n,n') Level
293 Kelvin Cross Sections

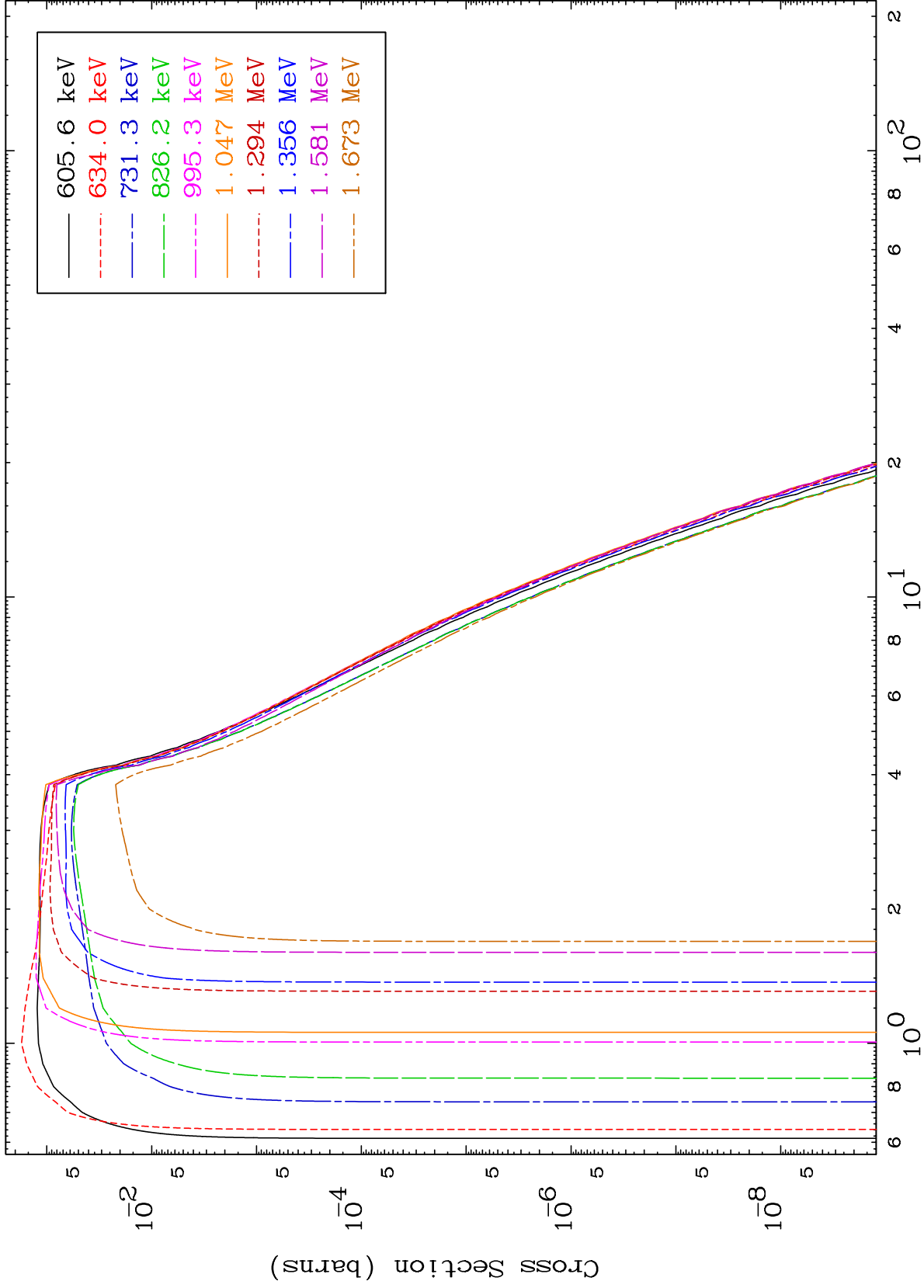
37-Rb-90



MAT 3741

(n,n') Level
293 Kelvin Cross Sections

37-Rb-90



8

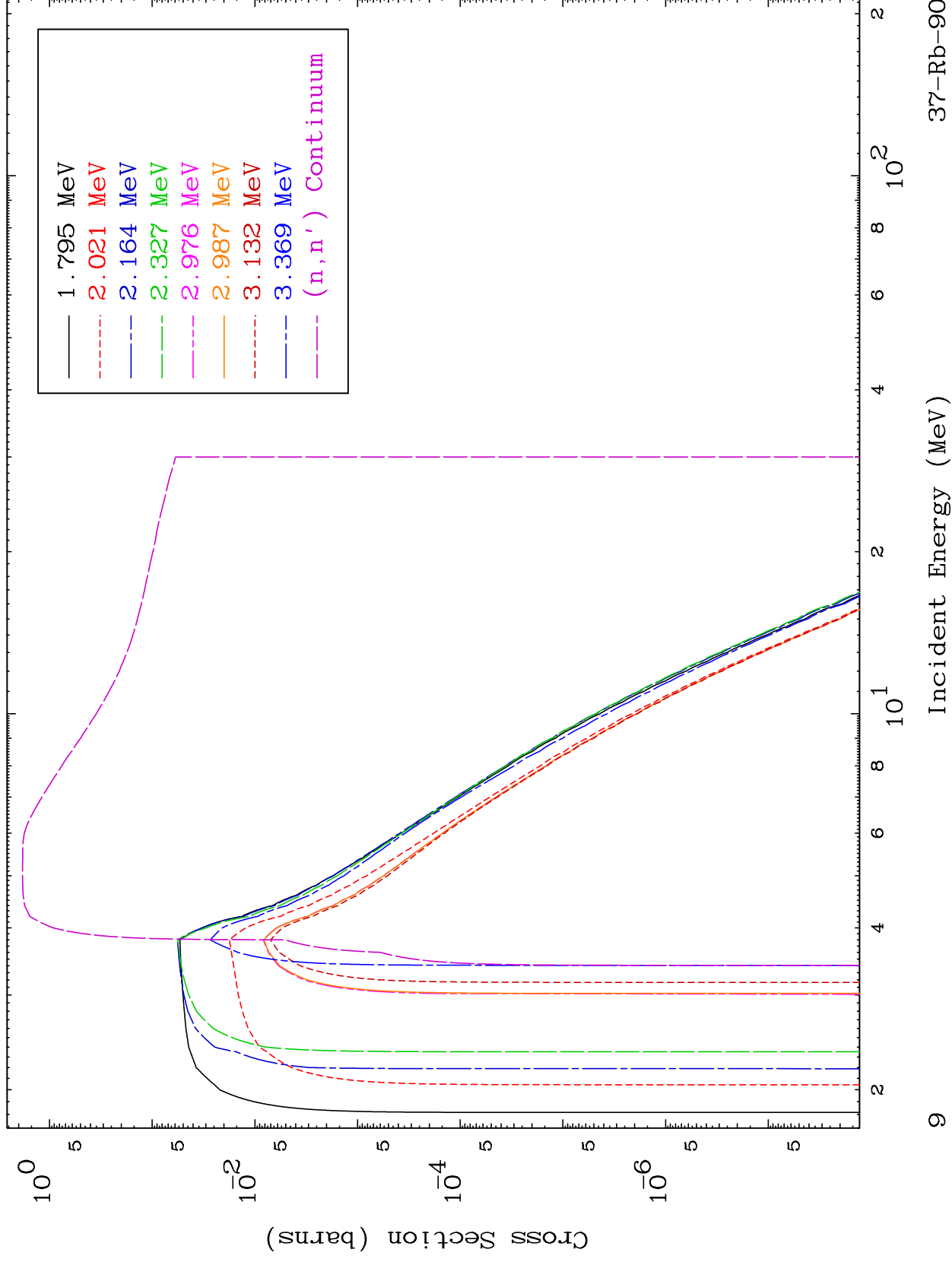
Incident Energy (MeV)

37-Rb-90

MAT 3741

37-Rb-90

(n,n') Level
293 Kelvin Cross Sections



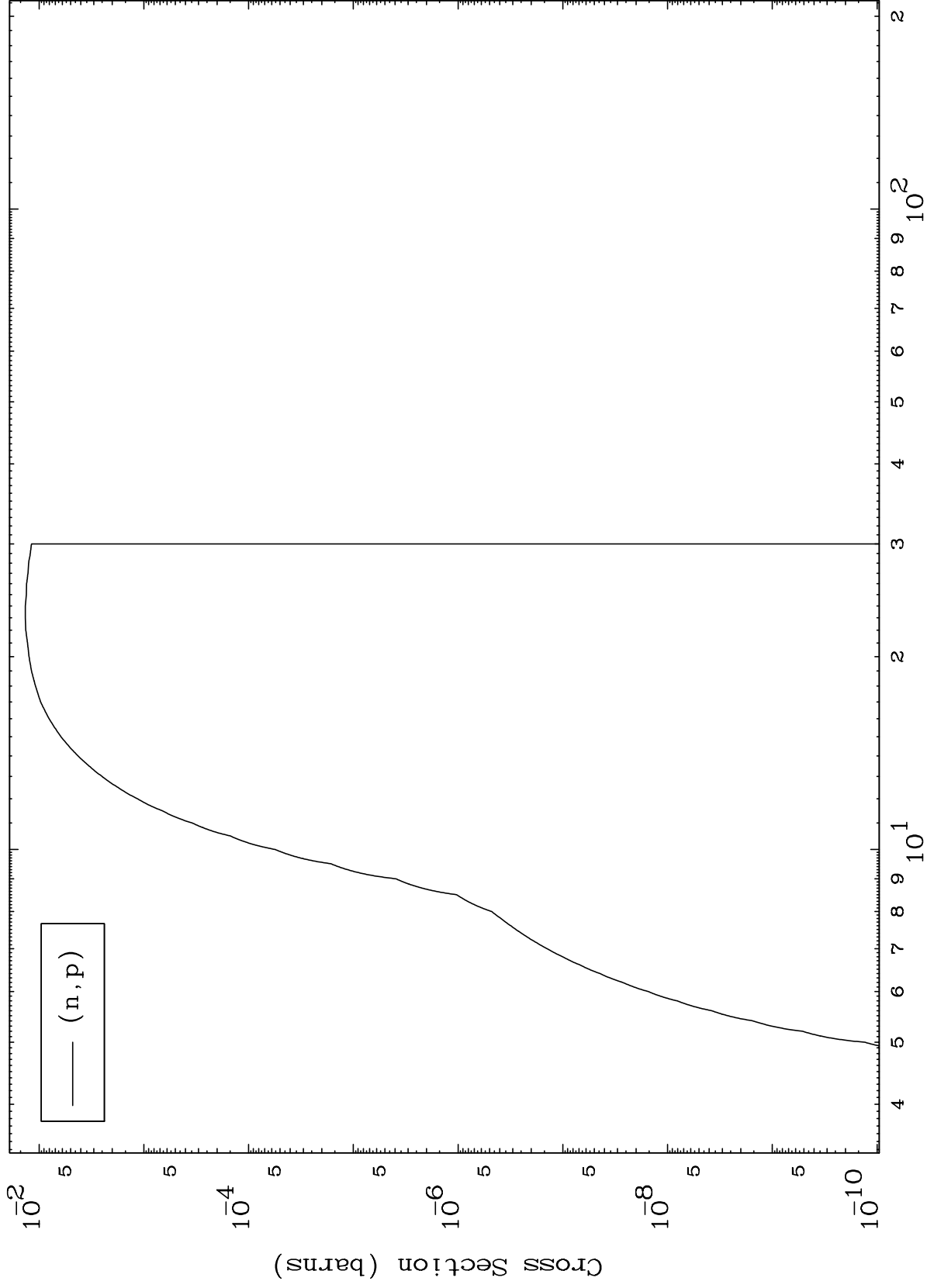
9

37-Rb-90

MAT 3741

(n,p) Levels
293 Kelvin Cross Sections

37-Rb-90



10

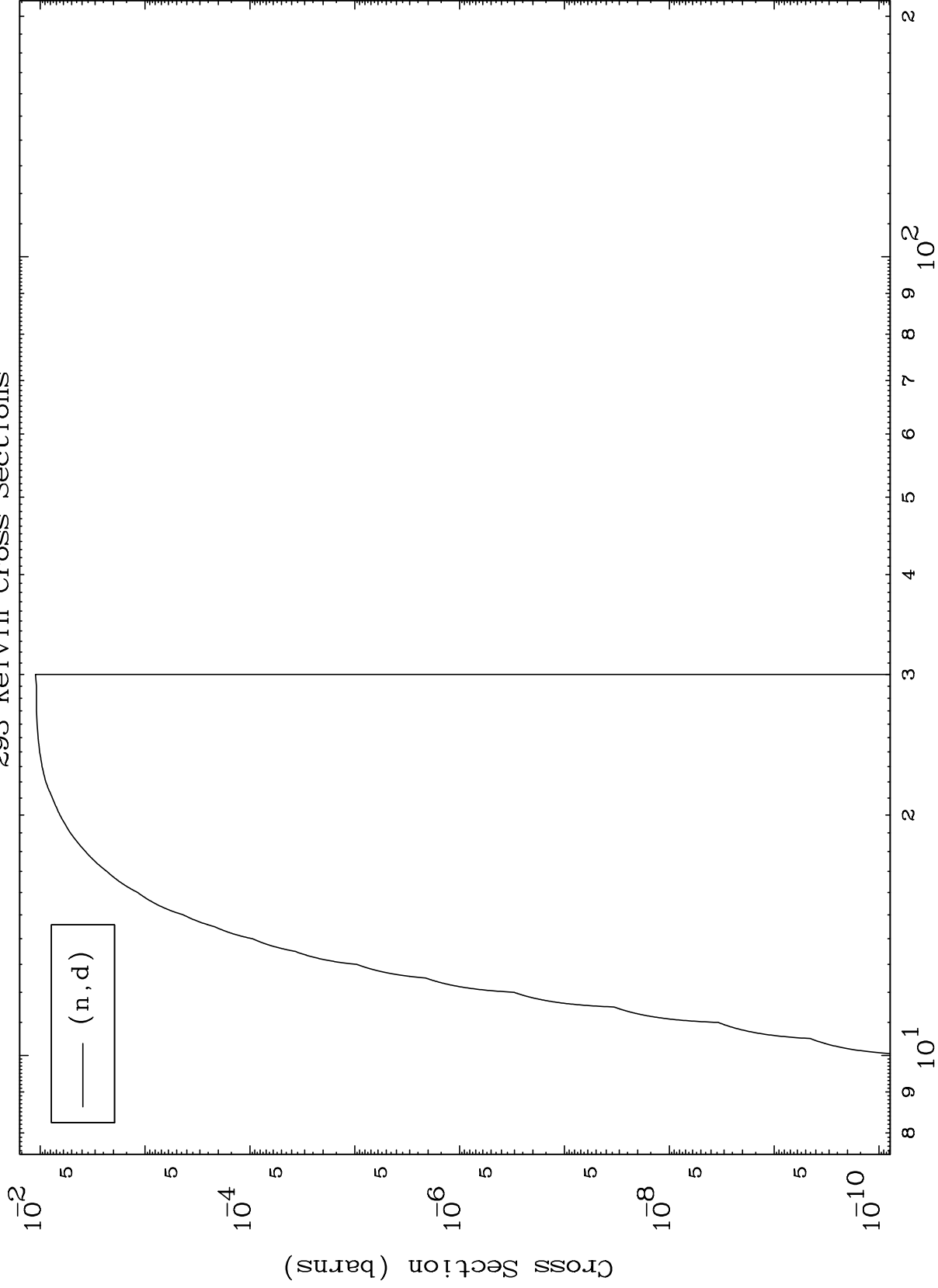
Incident Energy (MeV)

37-Rb-90

MAT 3741

(n,d) Levels
293 Kelvin Cross Sections

37-Rb-90



11

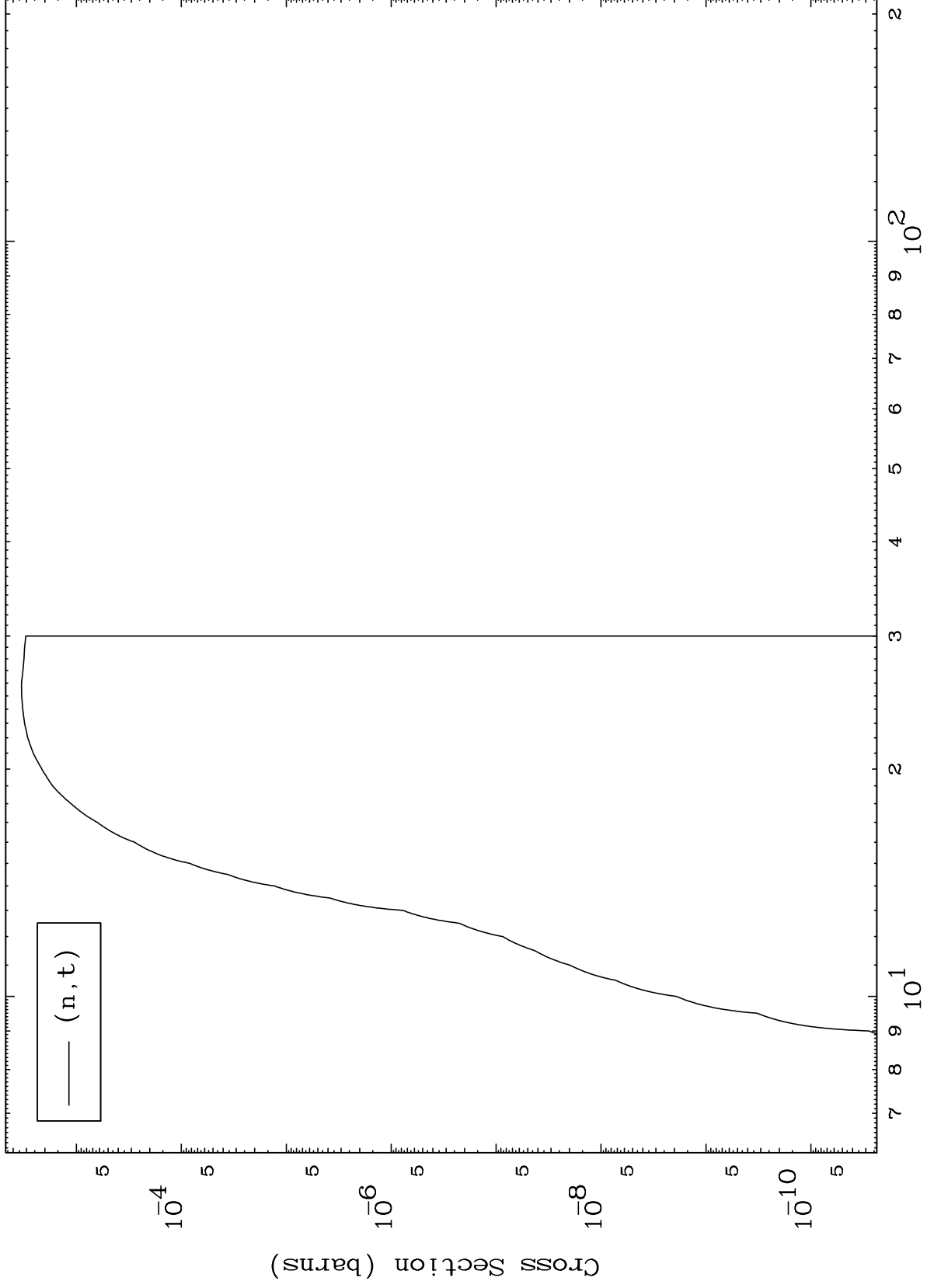
Incident Energy (MeV)

37-Rb-90

MAT 3741

(n,t) Levels
293 Kelvin Cross Sections

37-Rb-90



12

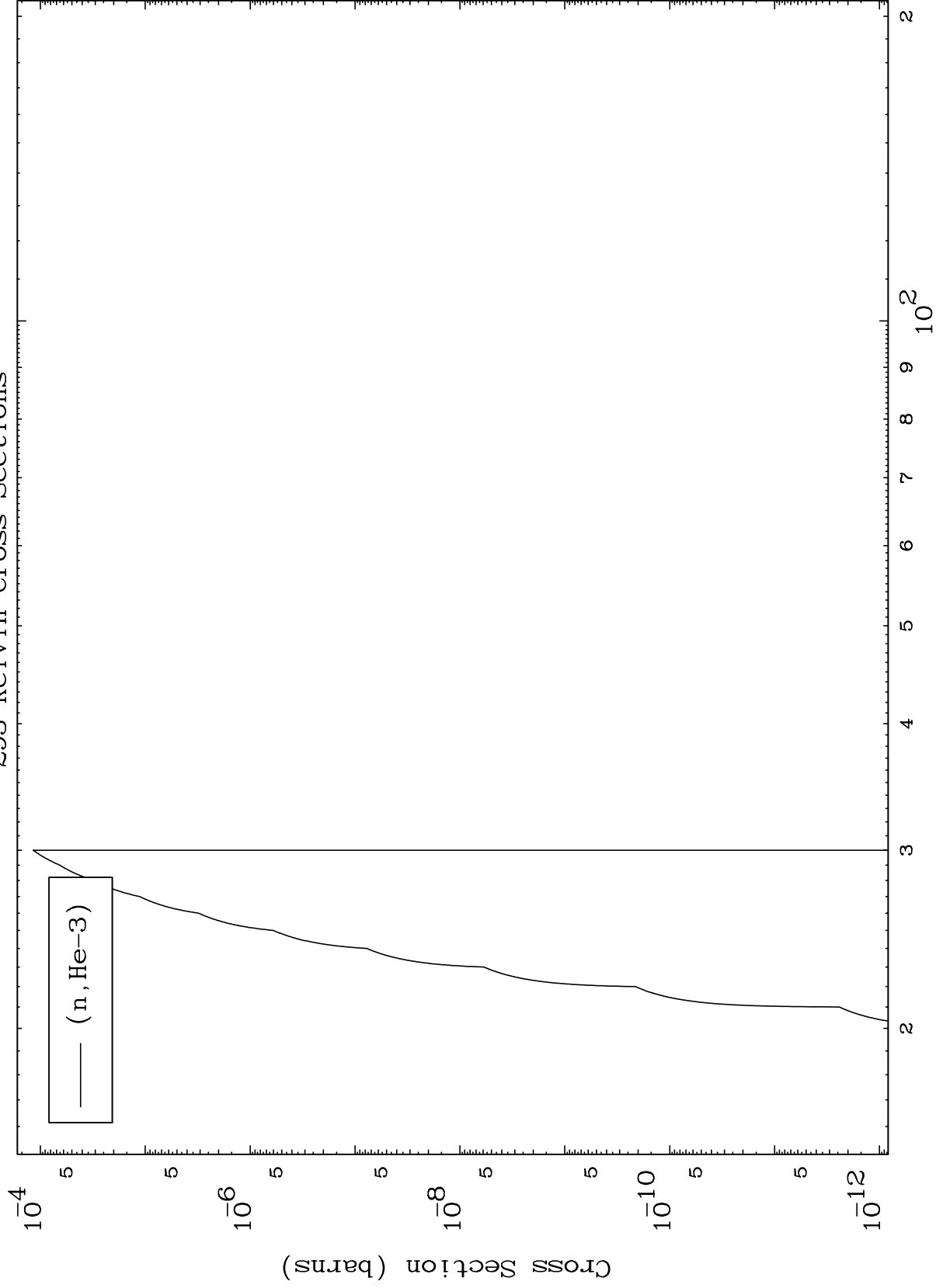
Incident Energy (MeV)

37-Rb-90

MAT 3741

(n,He3) Levels
293 Kelvin Cross Sections

37-Rb-90



13

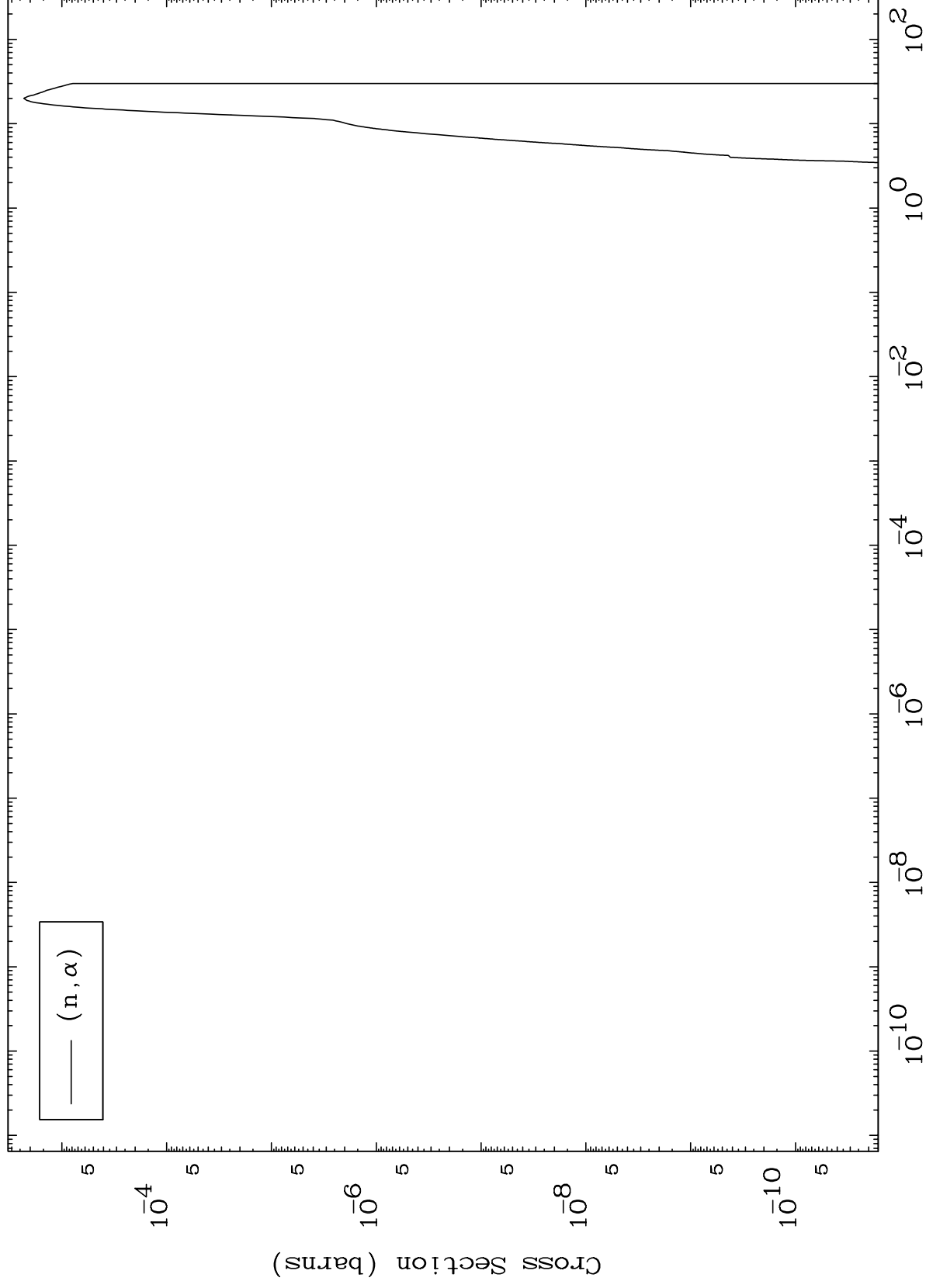
Incident Energy (MeV)

37-Rb-90

MAT 3741

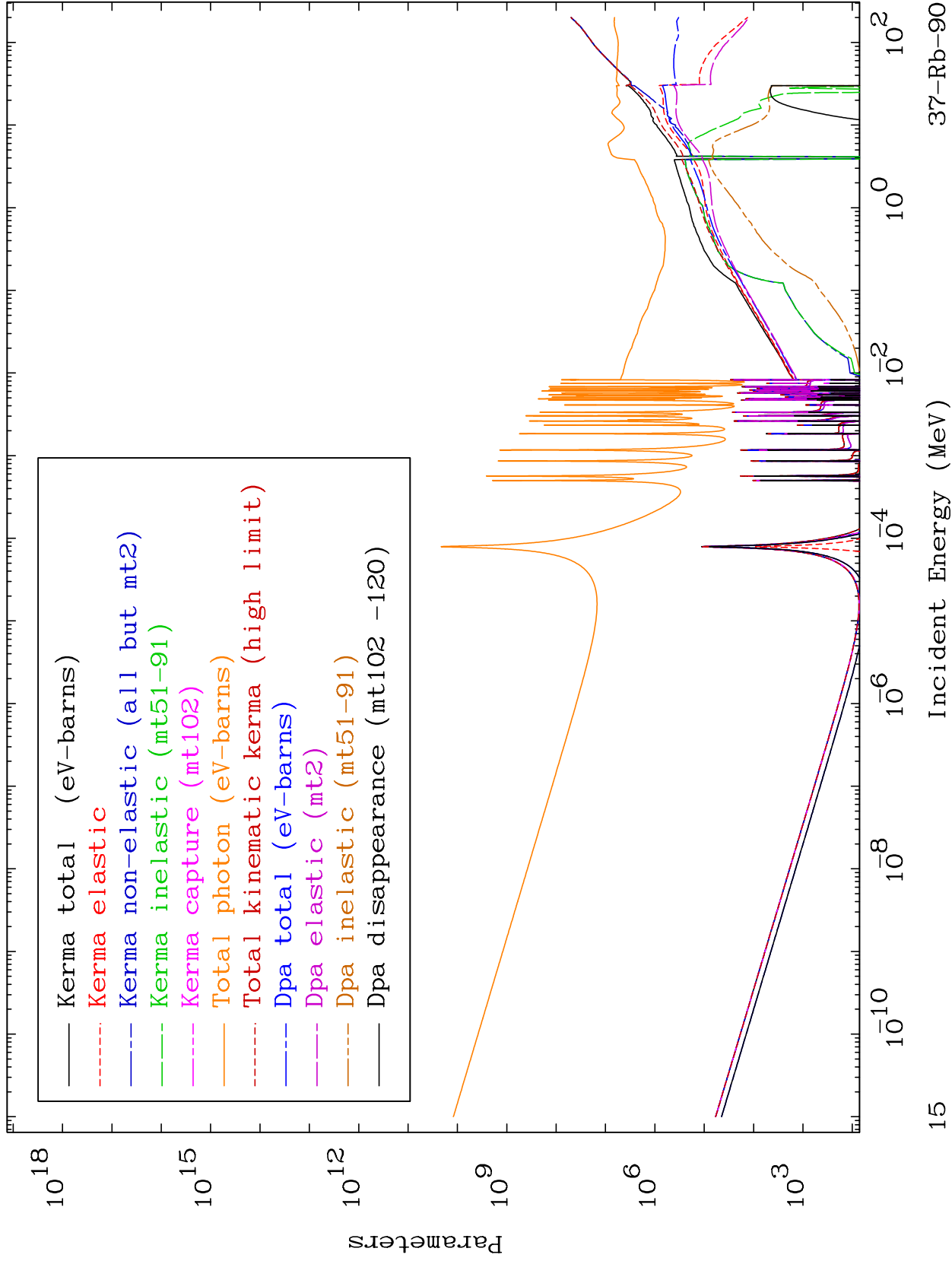
(n, α) Levels
293 Kelvin Cross Sections

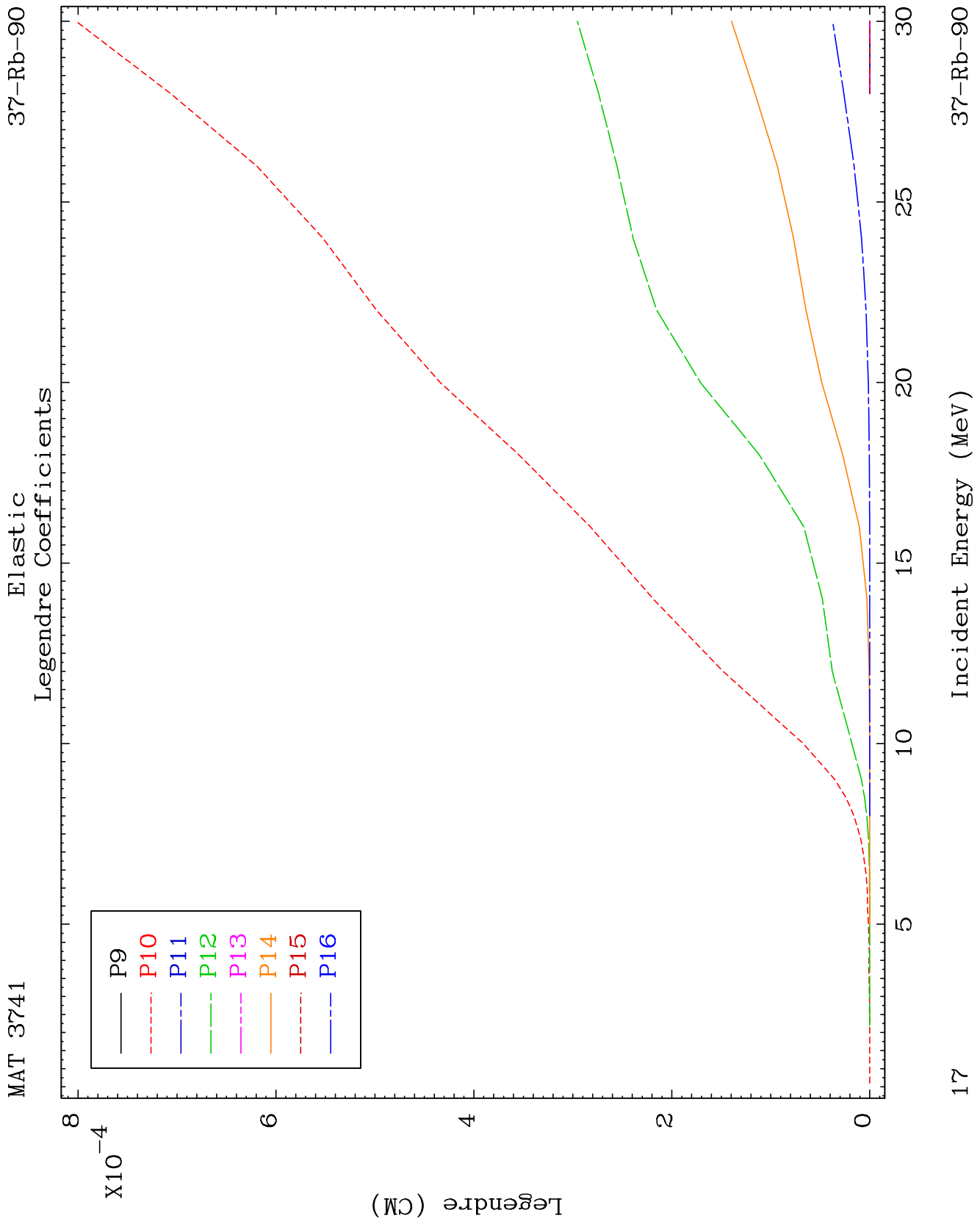
37-Rb-90



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37-Rb-90

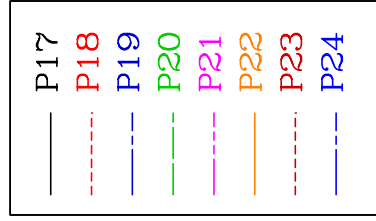




MAT 3741

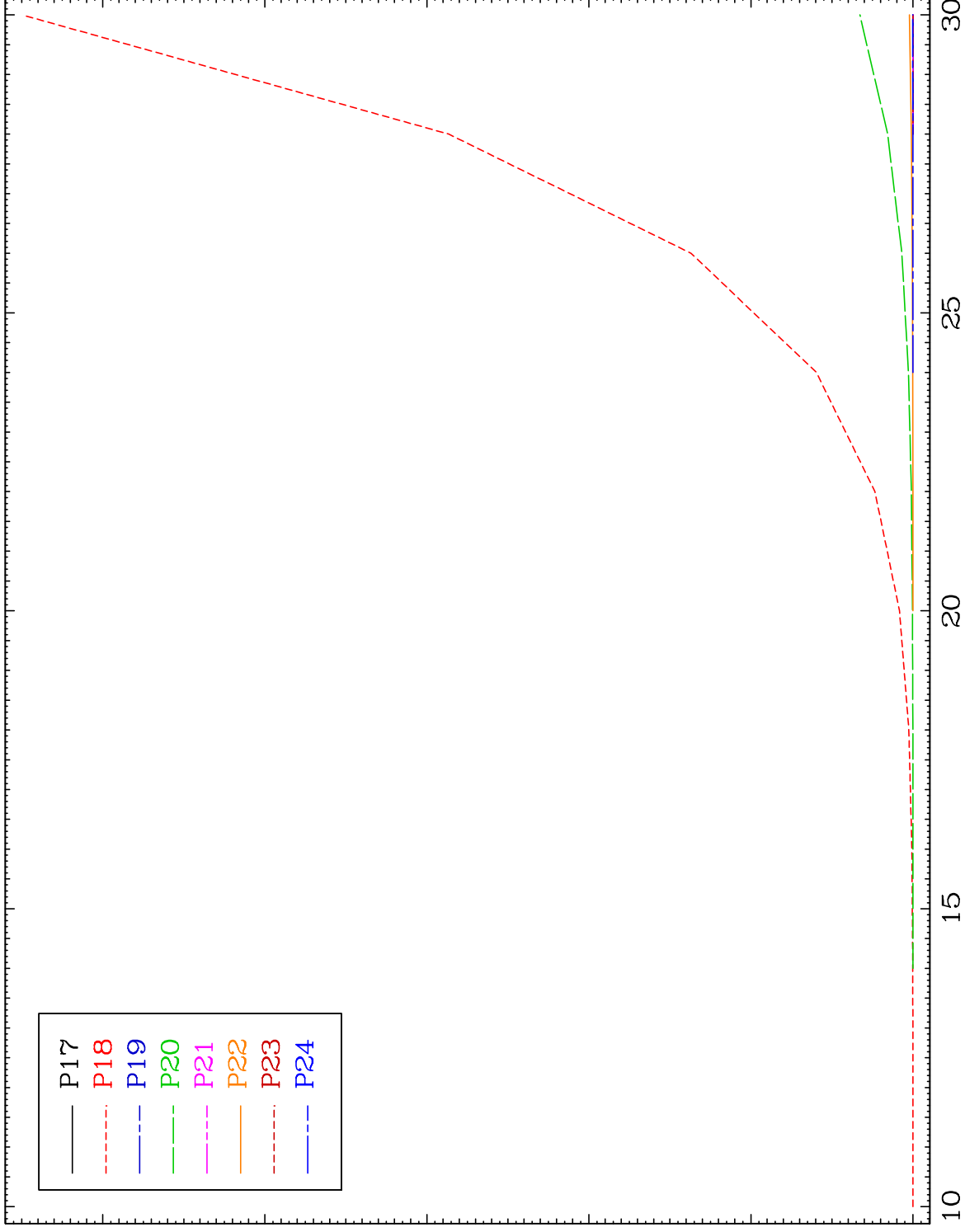
Elastic Legendre Coefficients

37-Rb-90



$\times 10^{-6}$

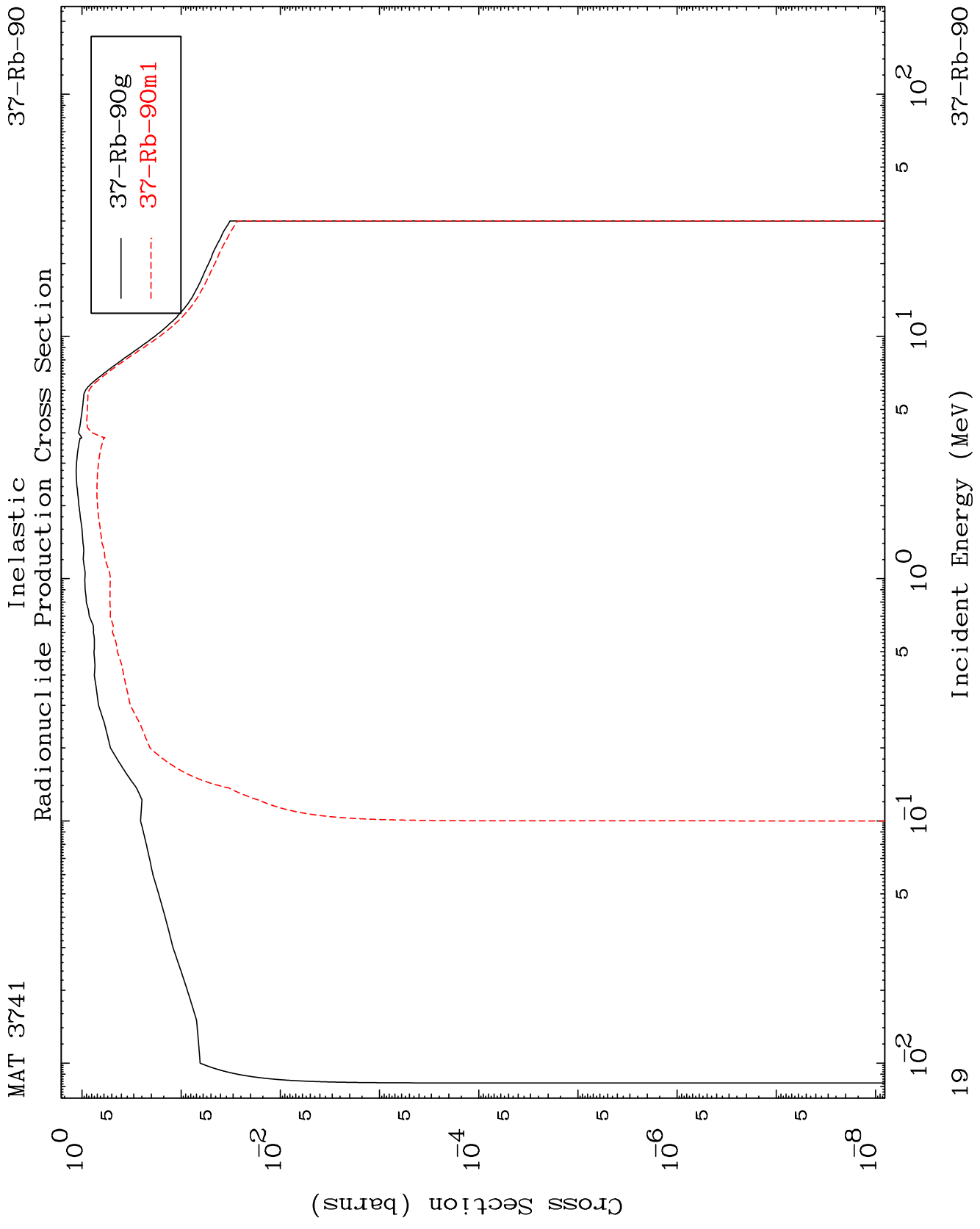
Legendre (CM)



18

Incident Energy (MeV)

37-Rb-90

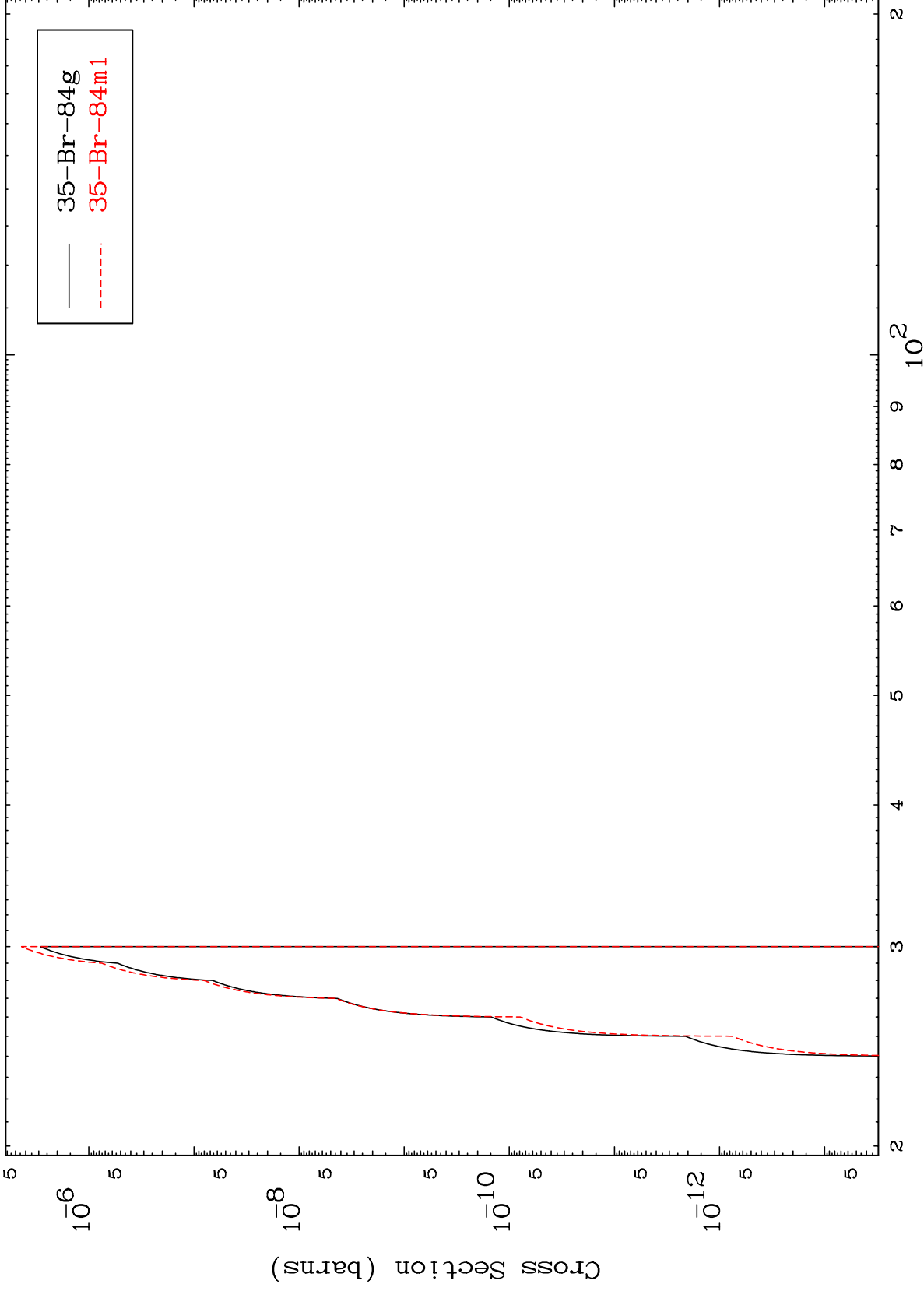


MAT 3741

$(n,3n) \alpha$

37-Rb-90

Radionuclide Production Cross Section



20

Incident Energy (MeV)

37-Rb-90