

Program EVALPLOT  
(Version 2015-2)

by

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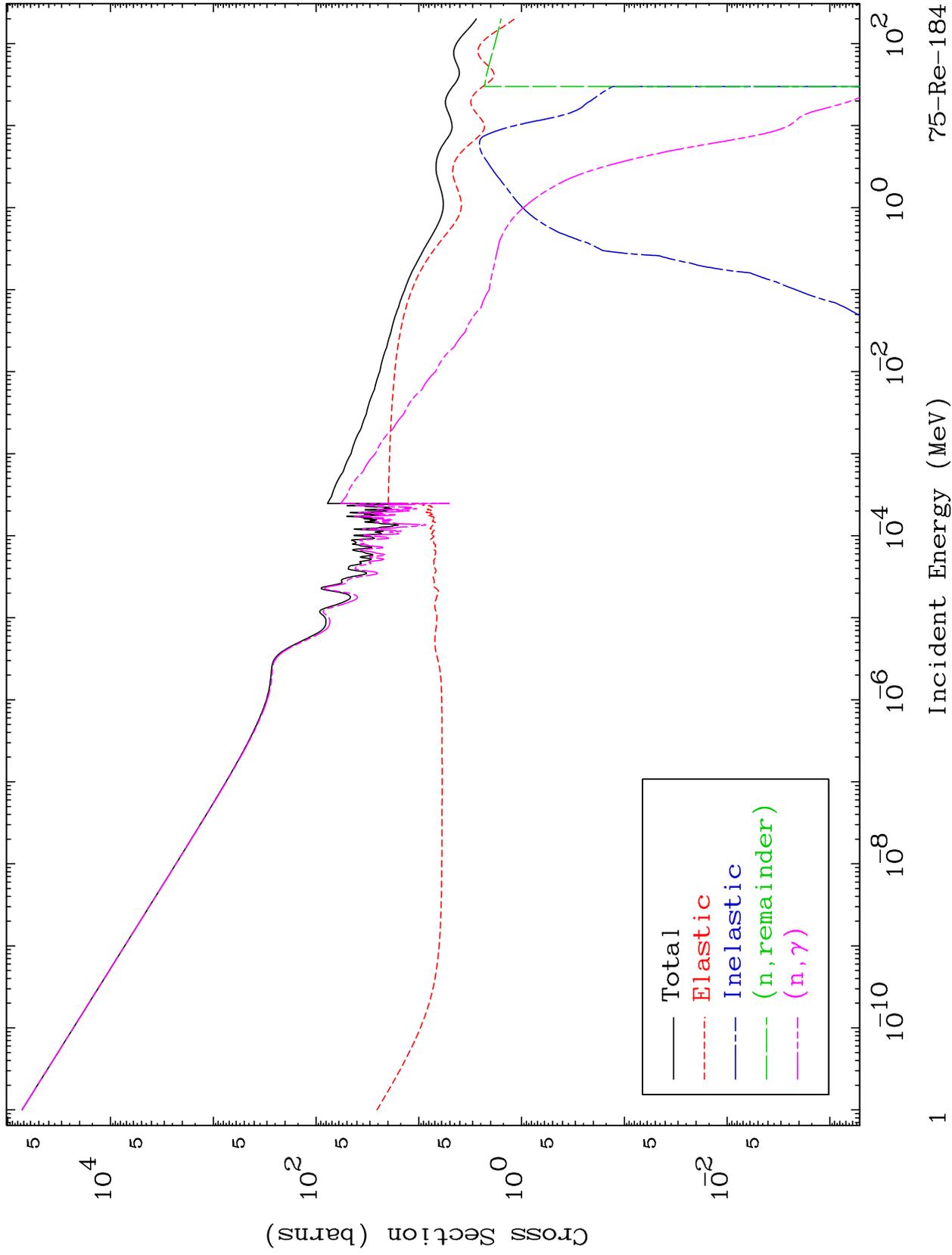
Press Mouse Button to Start

MAT 7523

Major

293 Kelvin Cross Sections

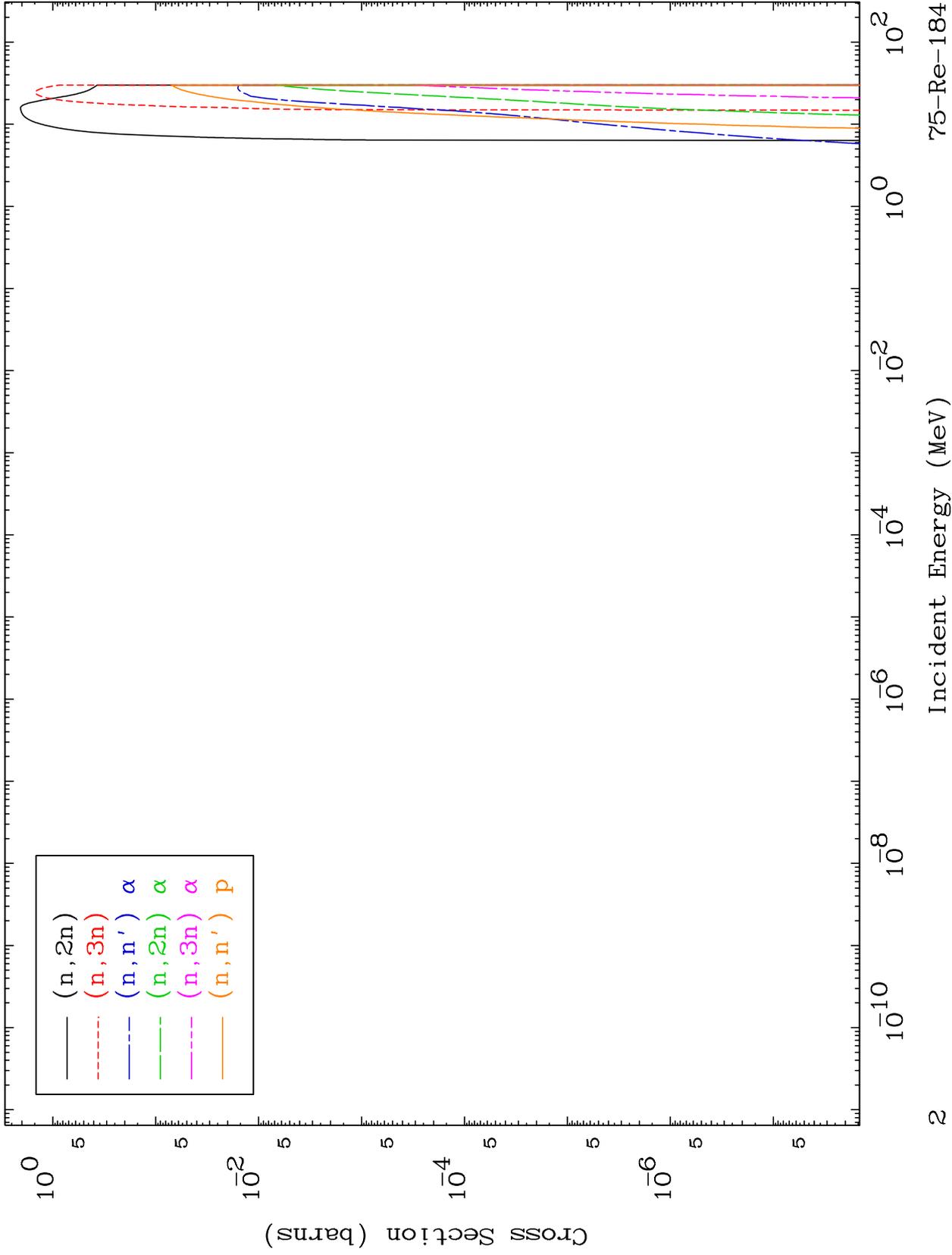
75-Re-184



MAT 7523

Neutron Production  
293 Kelvin Cross Sections

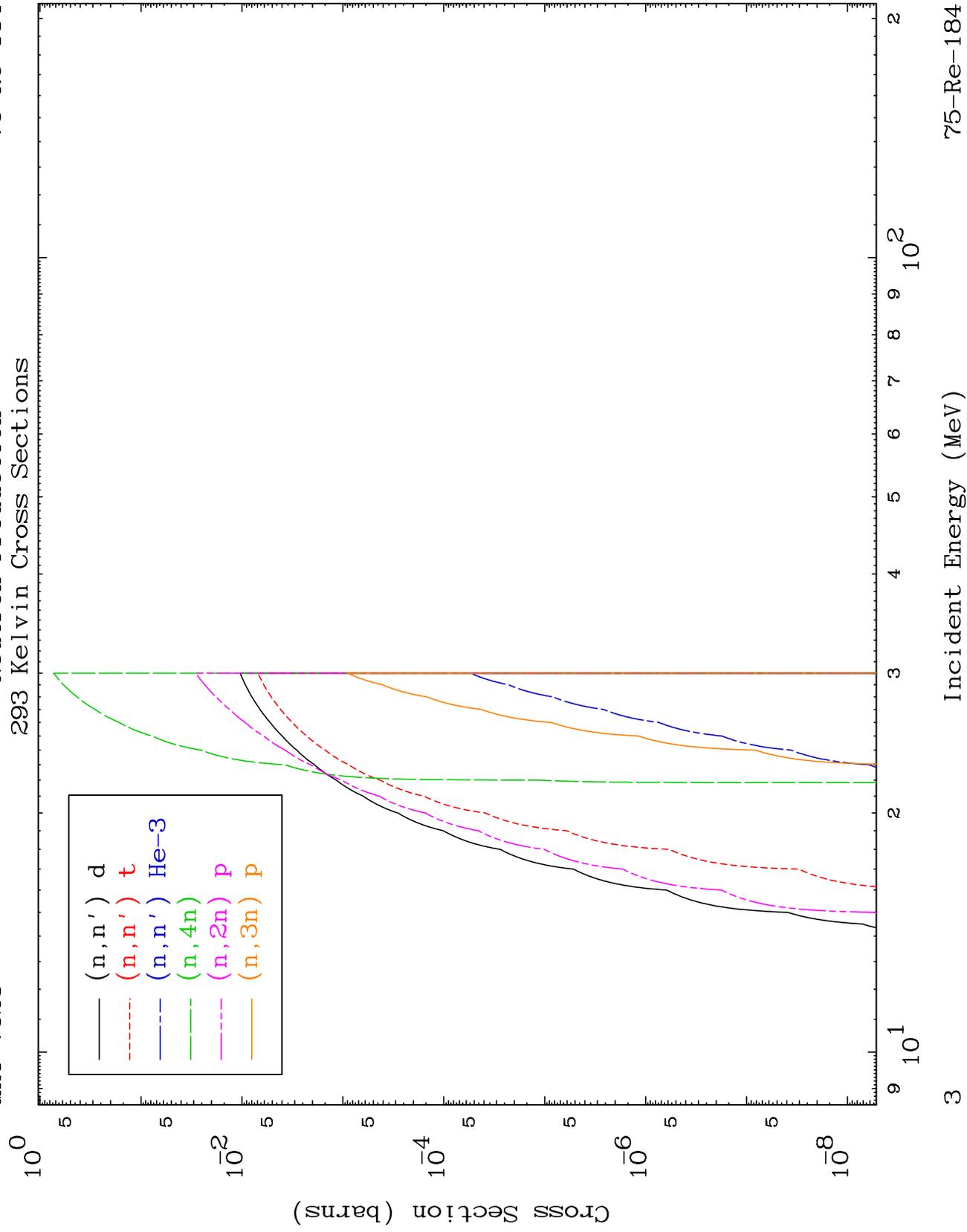
75-Re-184



MAT 7523

Neutron Production  
293 Kelvin Cross Sections

75-Re-184



75-Re-184

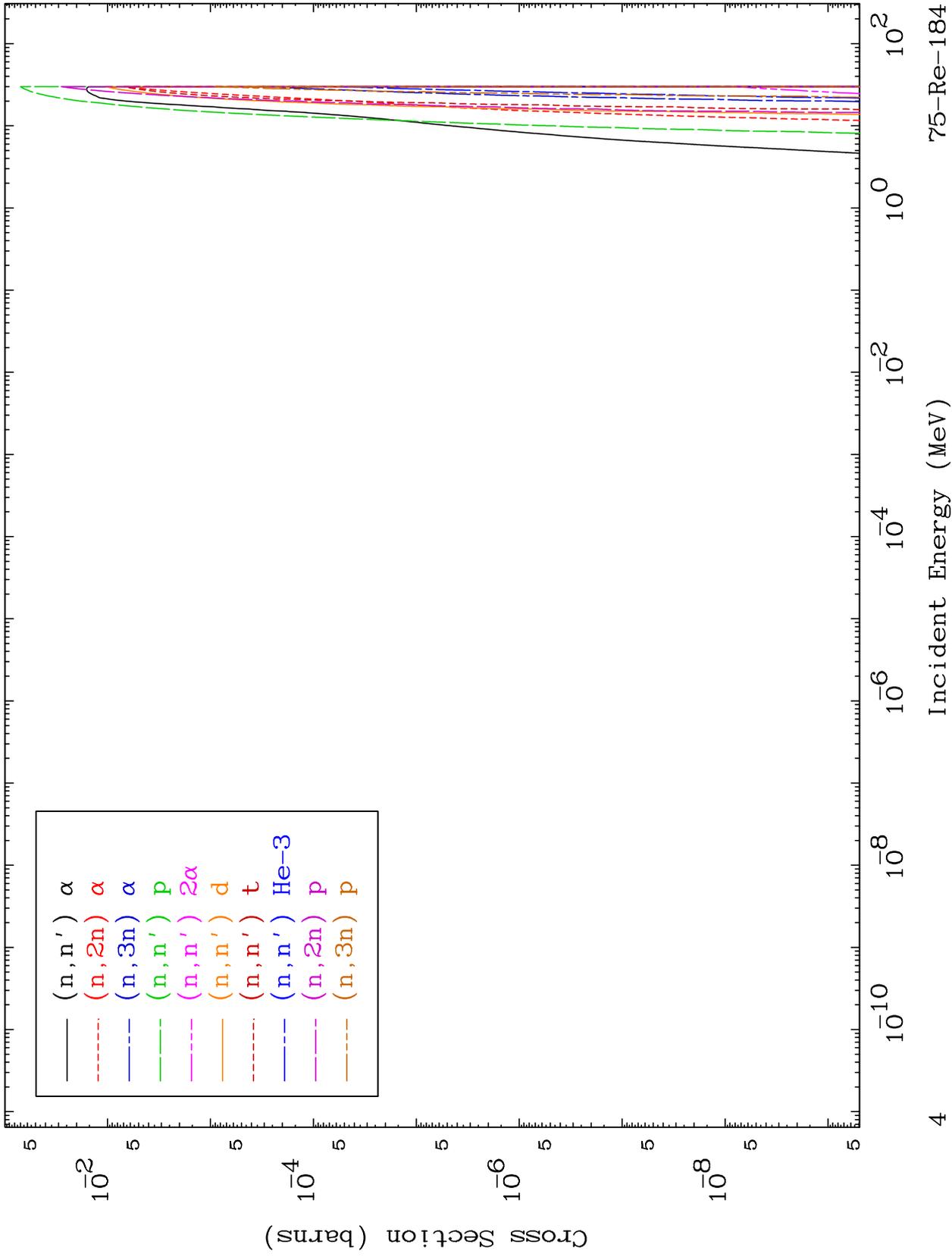
Incident Energy (MeV)

3

MAT 7523

Charged Particle  
293 Kelvin Cross Sections

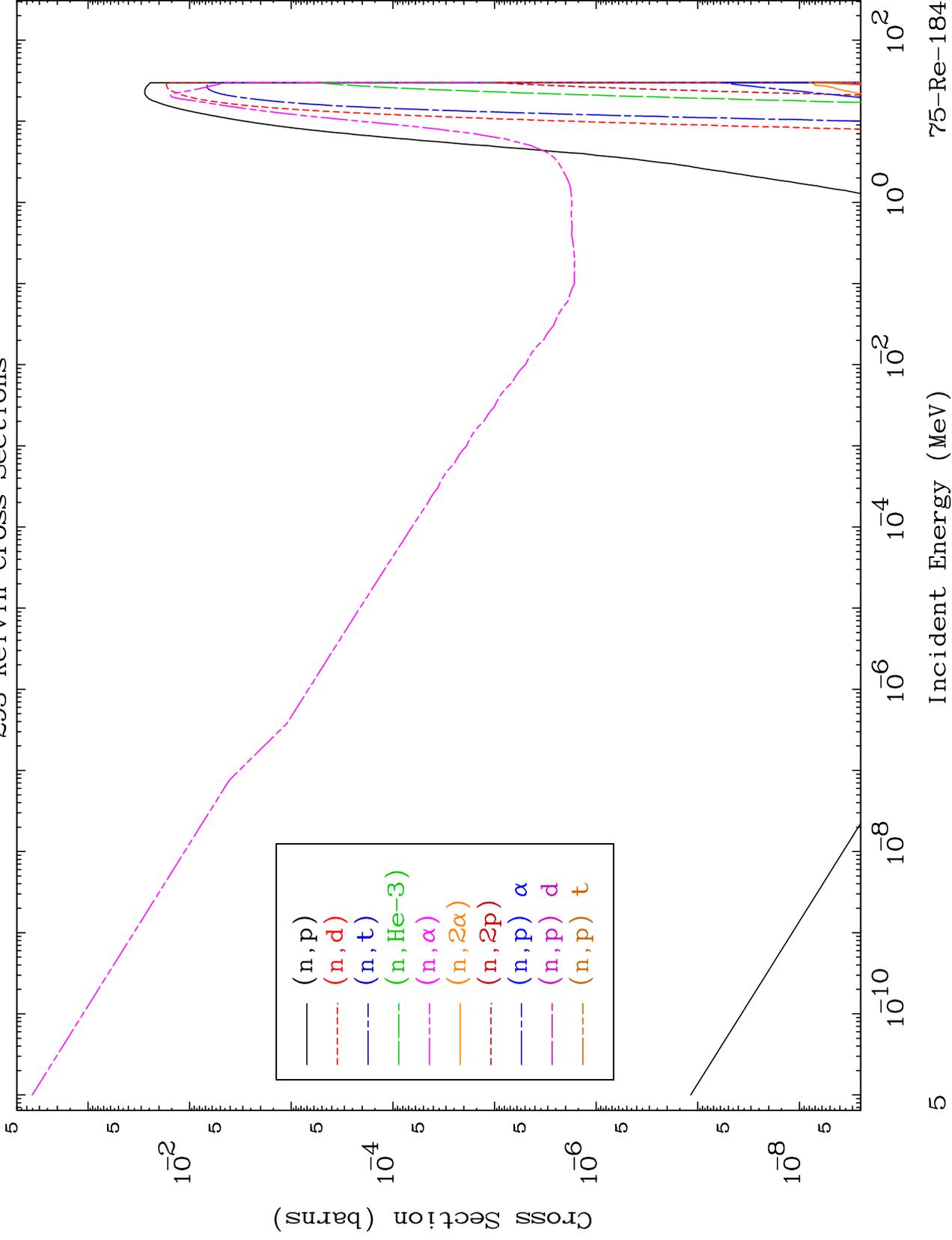
75-Re-184



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Charged Particle  
293 Kelvin Cross Sections

75-Re-184



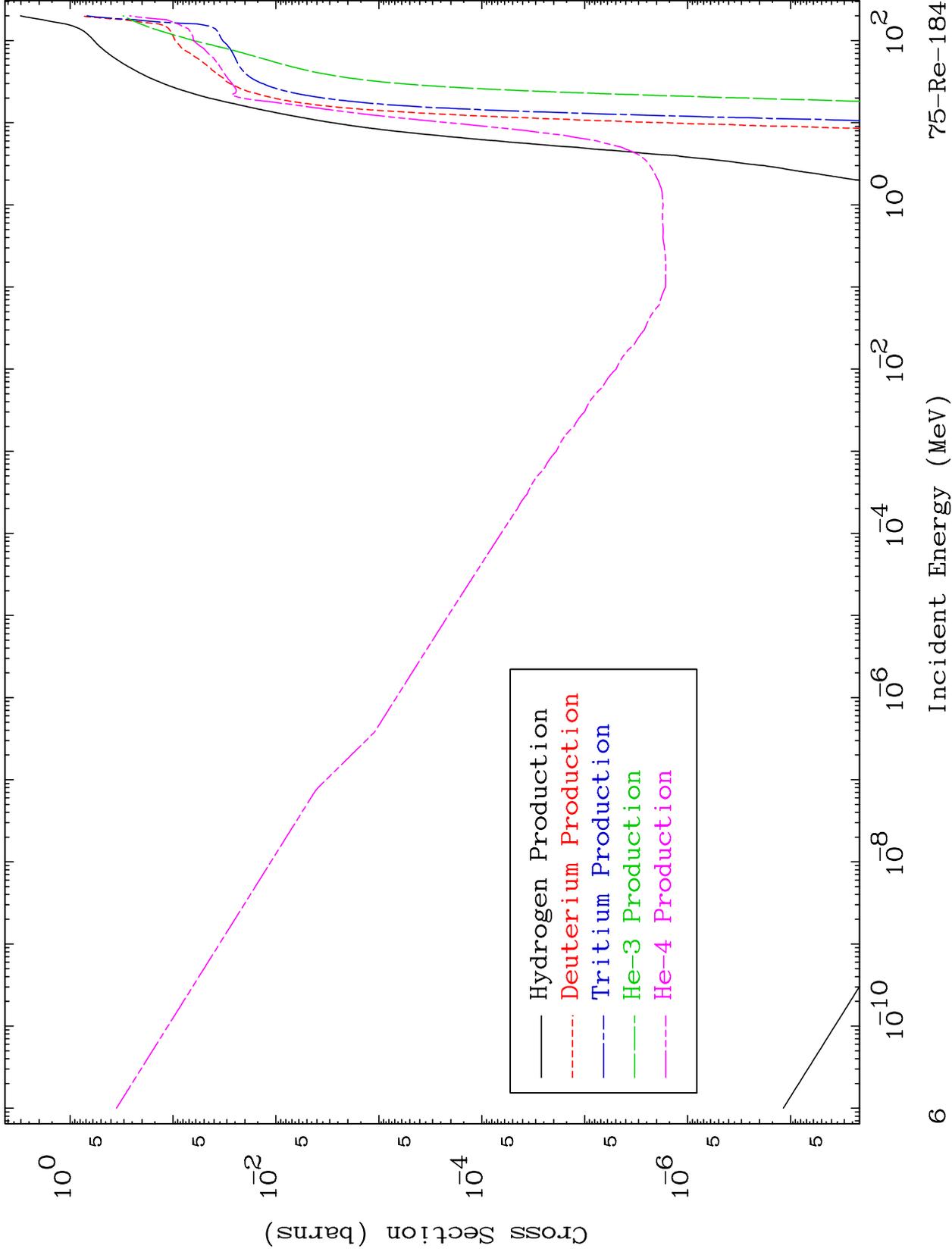
5

75-Re-184

MAT 7523

Particle Production  
293 Kelvin Cross Sections

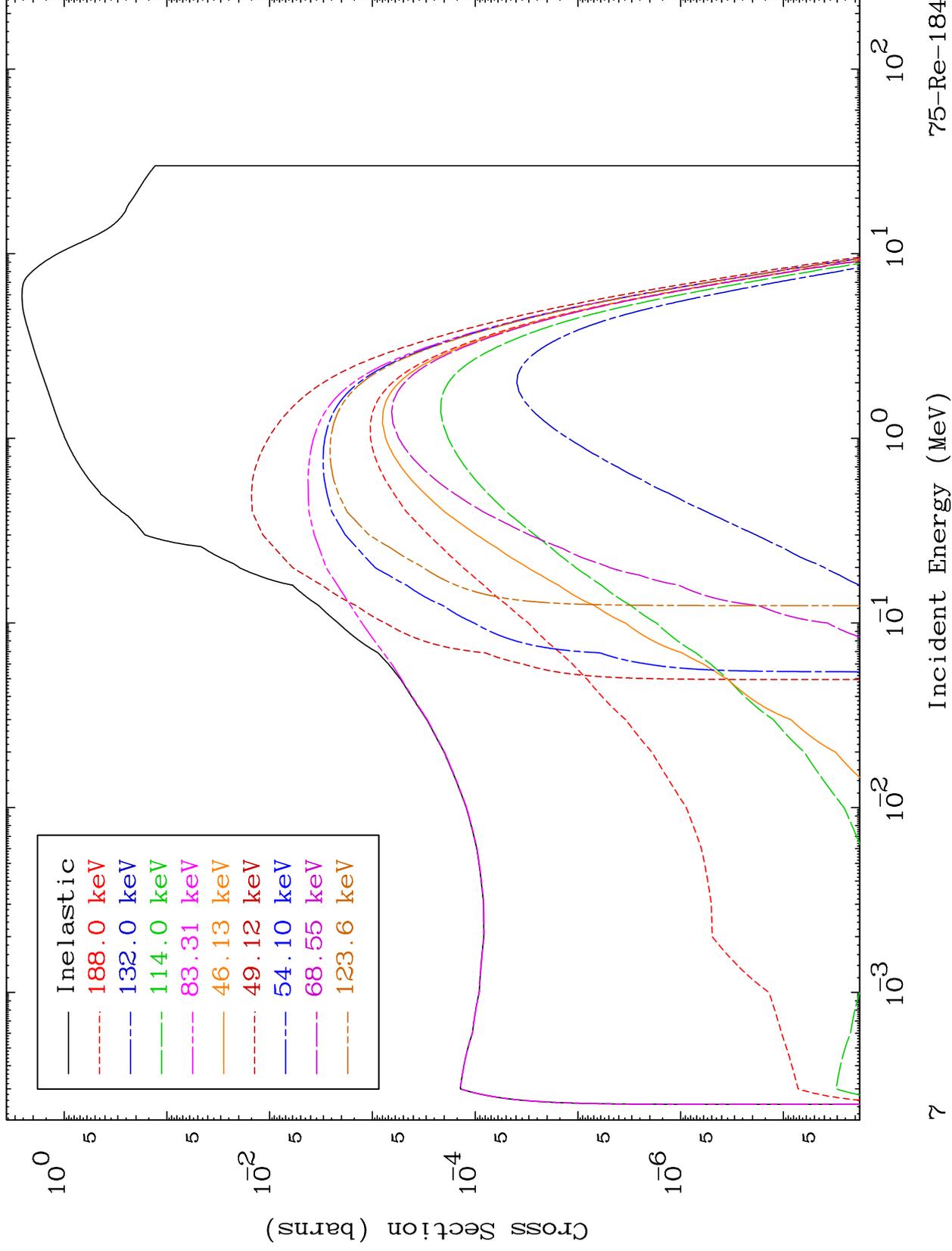
75-Re-184



MAT 7523

(n,n') Level  
293 Kelvin Cross Sections

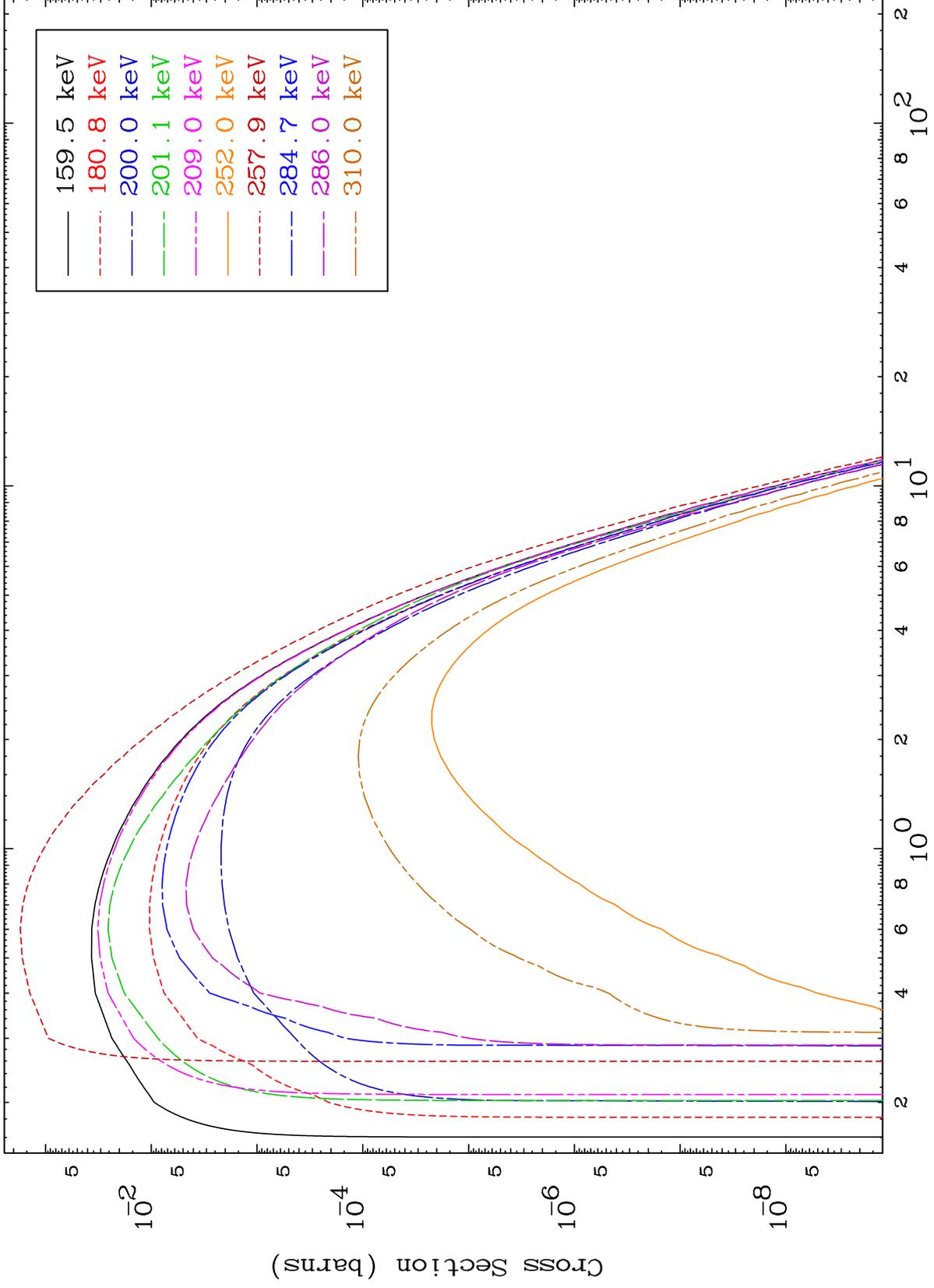
75-Re-184



MAT 7523

(n,n') Level  
293 Kelvin Cross Sections

75-Re-184



8

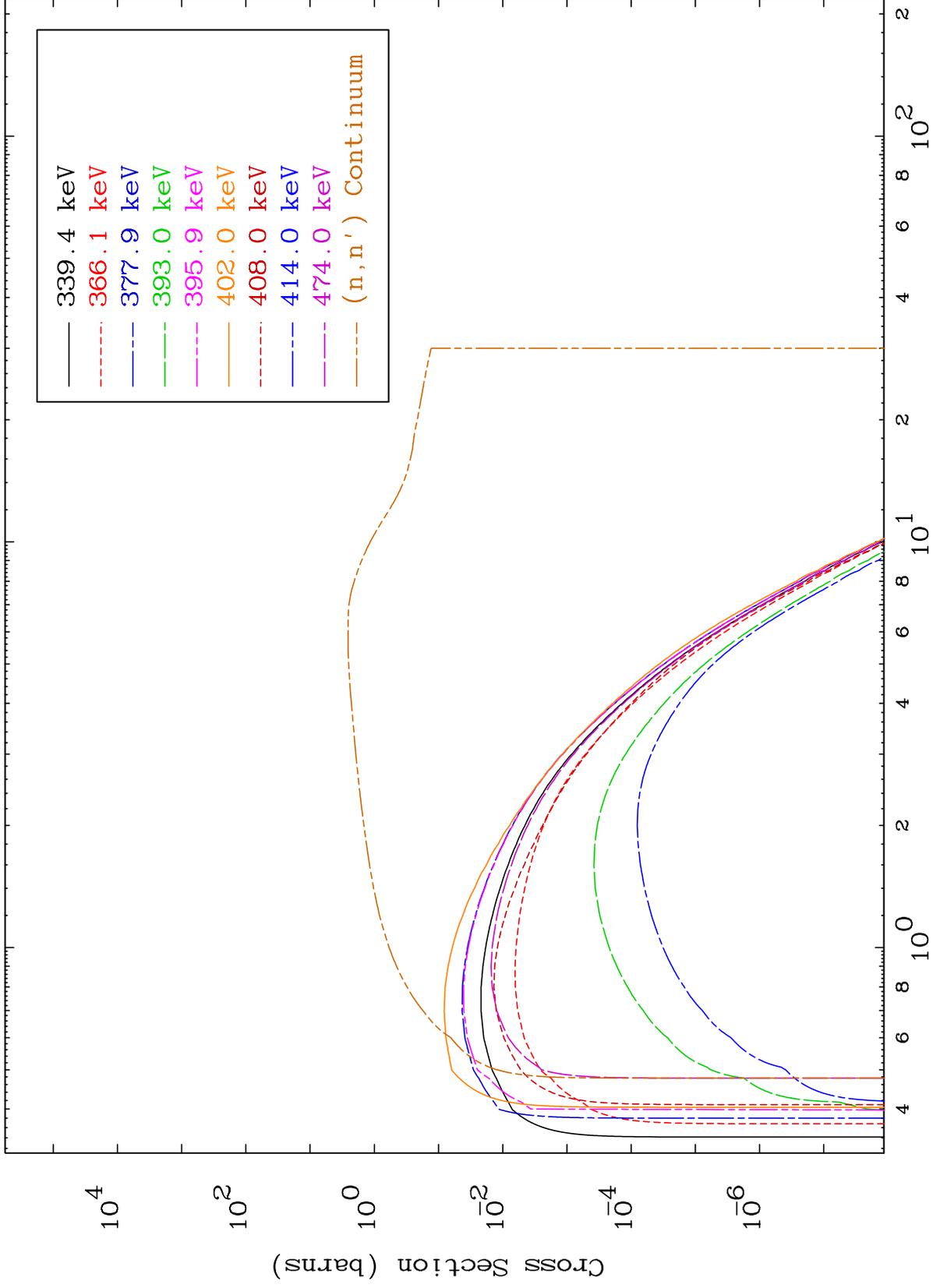
Incident Energy (MeV)

75-Re-184

MAT 7523

(n,n') Level  
293 Kelvin Cross Sections

75-Re-184



9

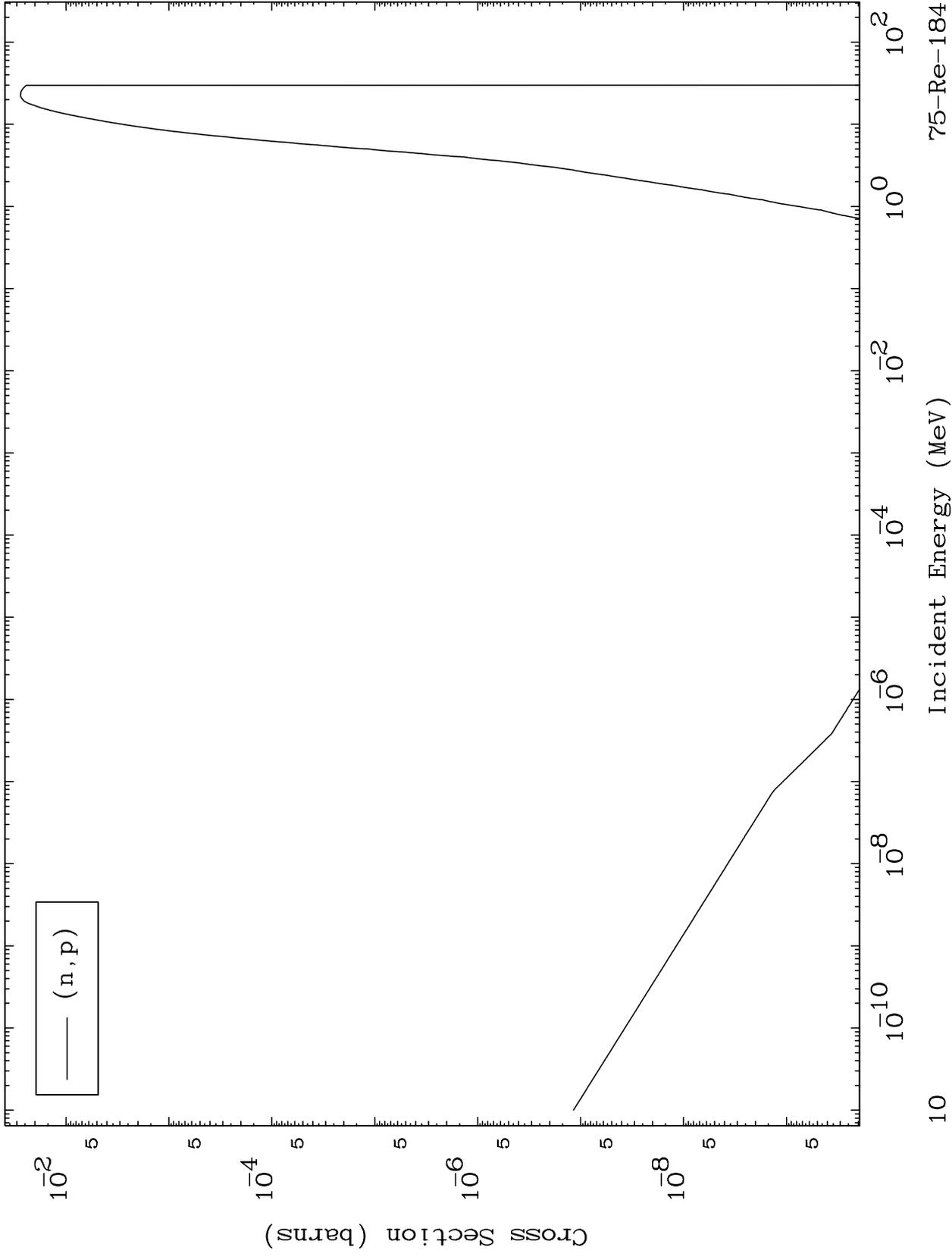
Incident Energy (MeV)

75-Re-184

MAT 7523

(n,p) Levels  
293 Kelvin Cross Sections

75-Re-184



10

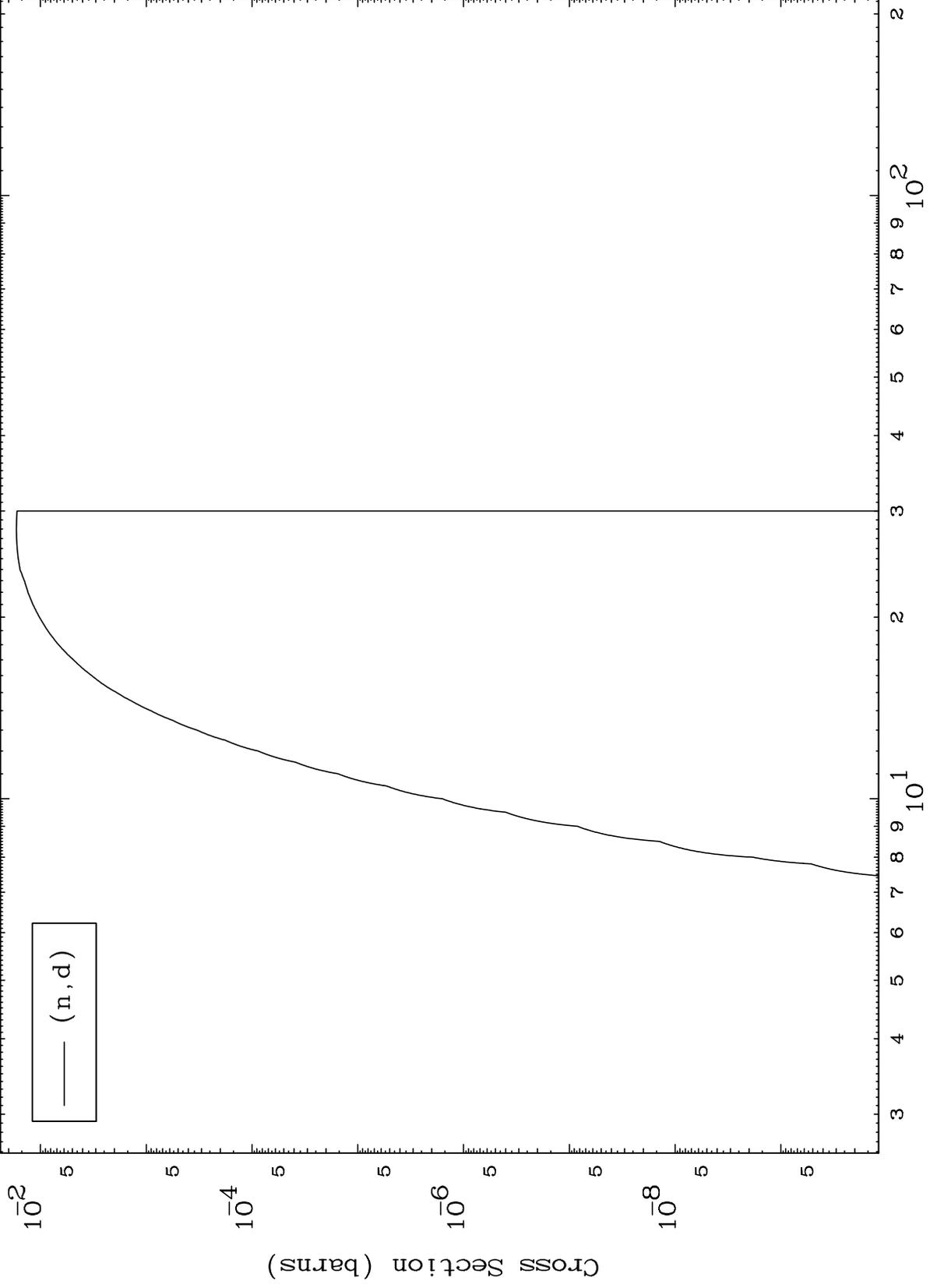
Incident Energy (MeV)

75-Re-184

MAT 7523

(n,d) Levels  
293 Kelvin Cross Sections

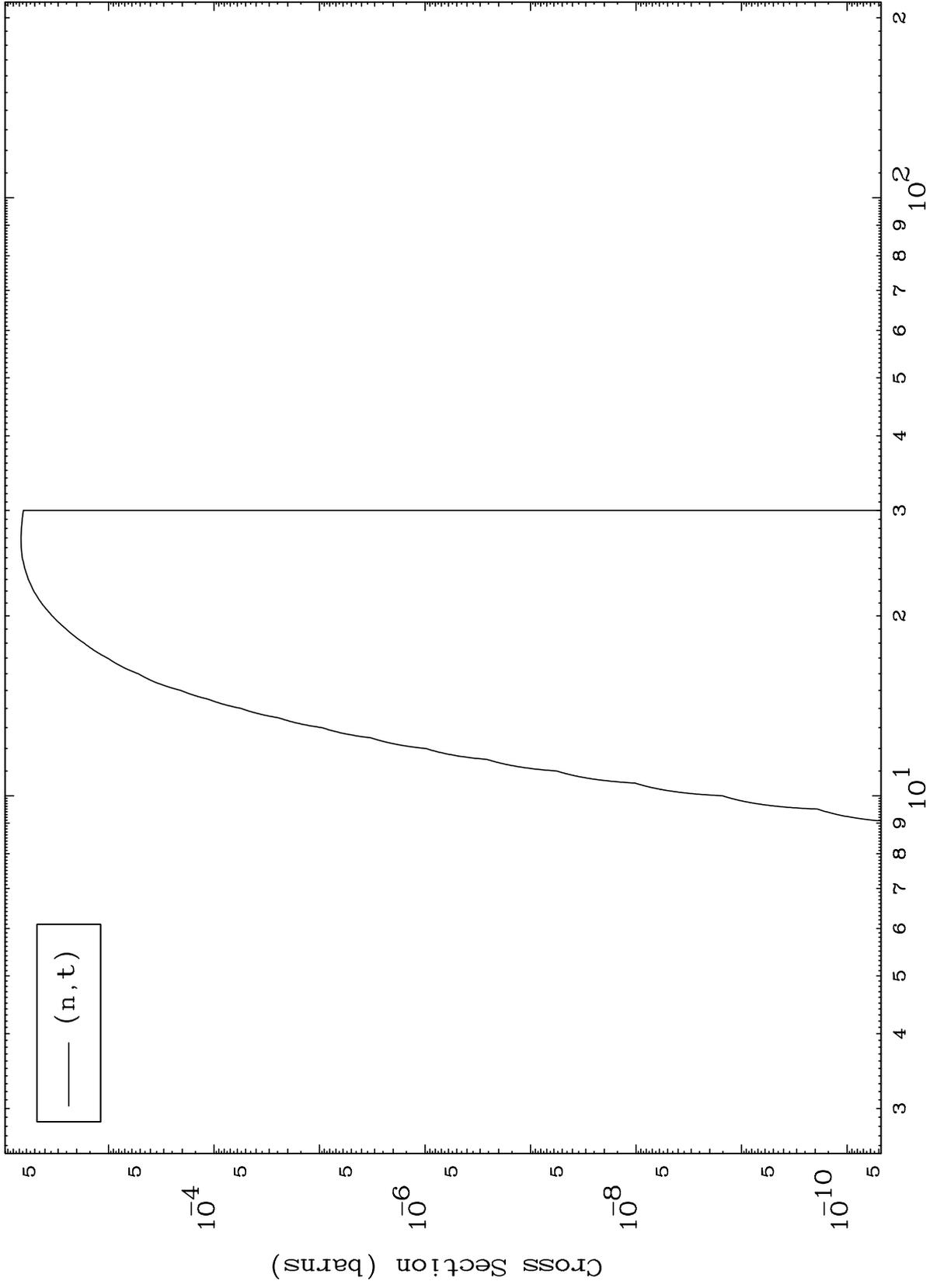
75-Re-184



MAT 7523

(n,t) Levels  
293 Kelvin Cross Sections

75-Re-184



(n,t)

12

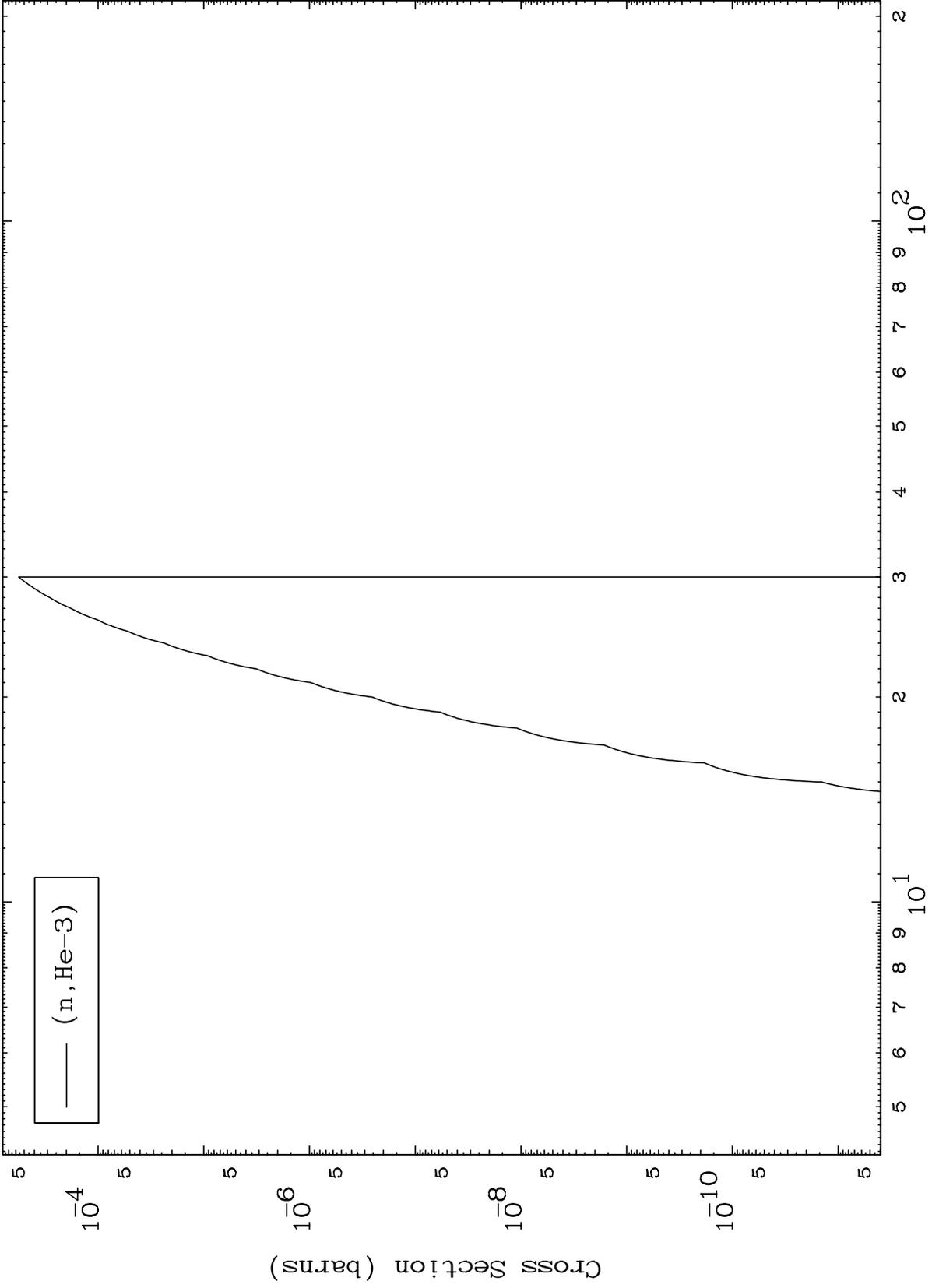
Incident Energy (MeV)

75-Re-184

MAT 7523

(n,He3) Levels  
293 Kelvin Cross Sections

75-Re-184



13

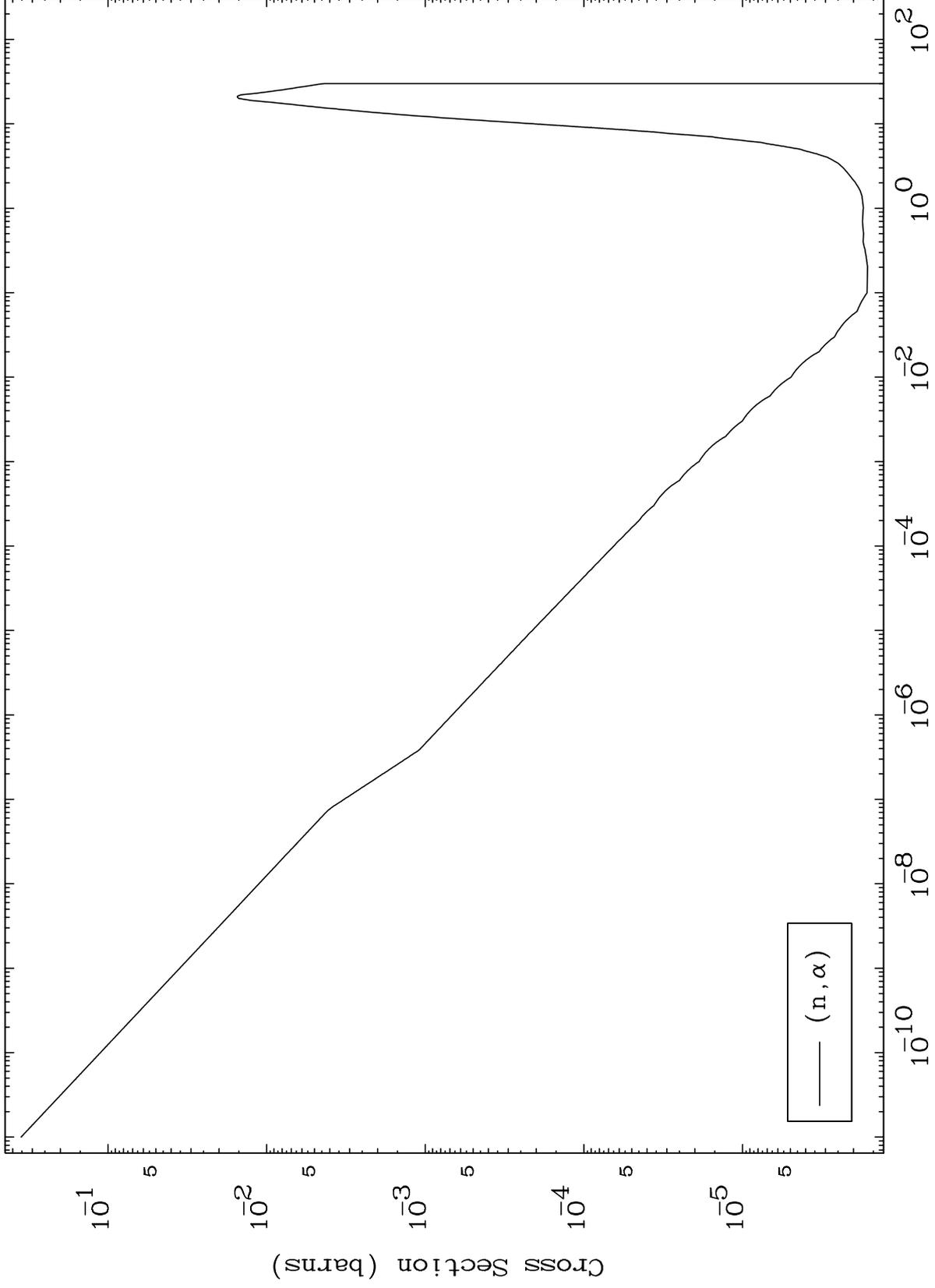
Incident Energy (MeV)

75-Re-184

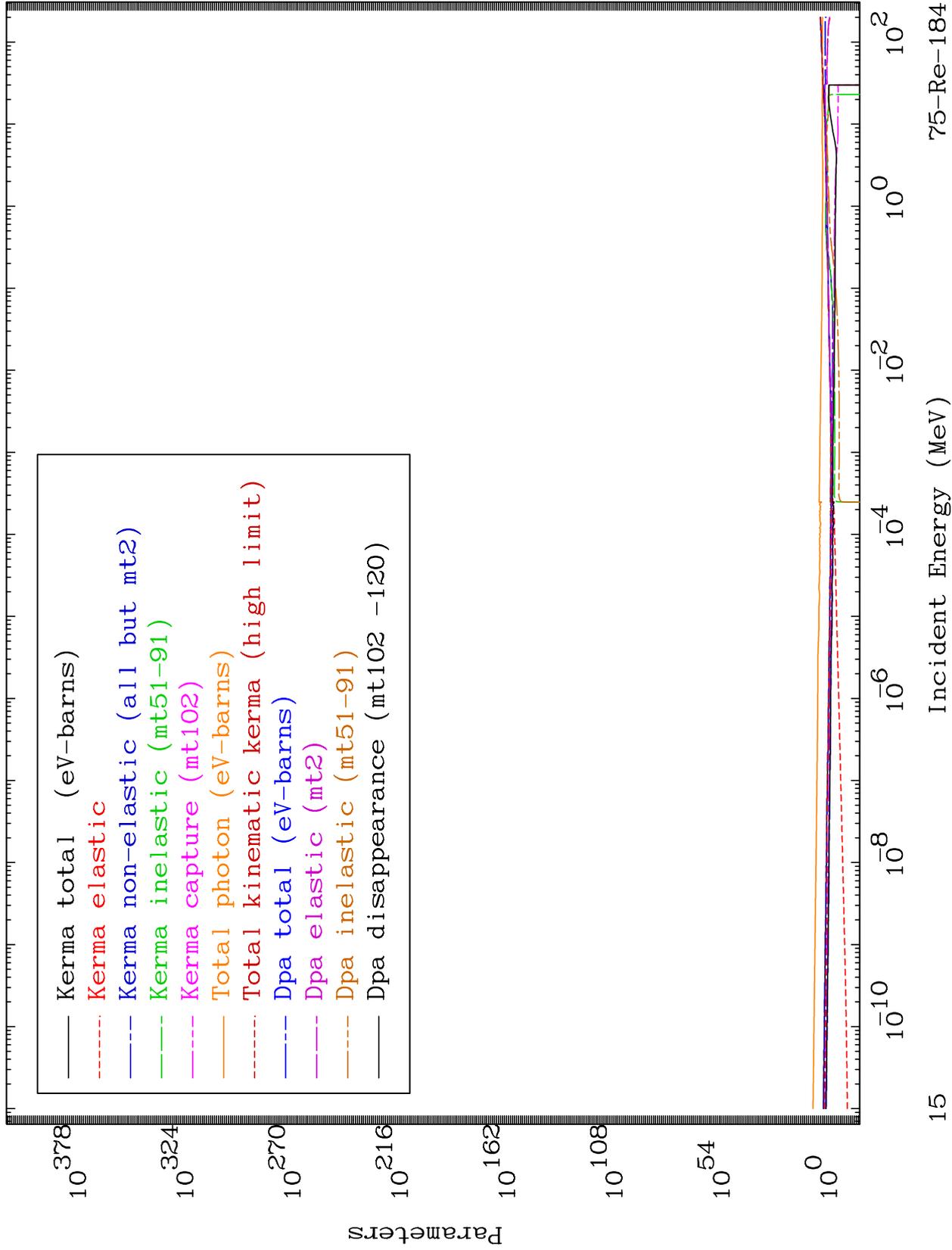
MAT 7523

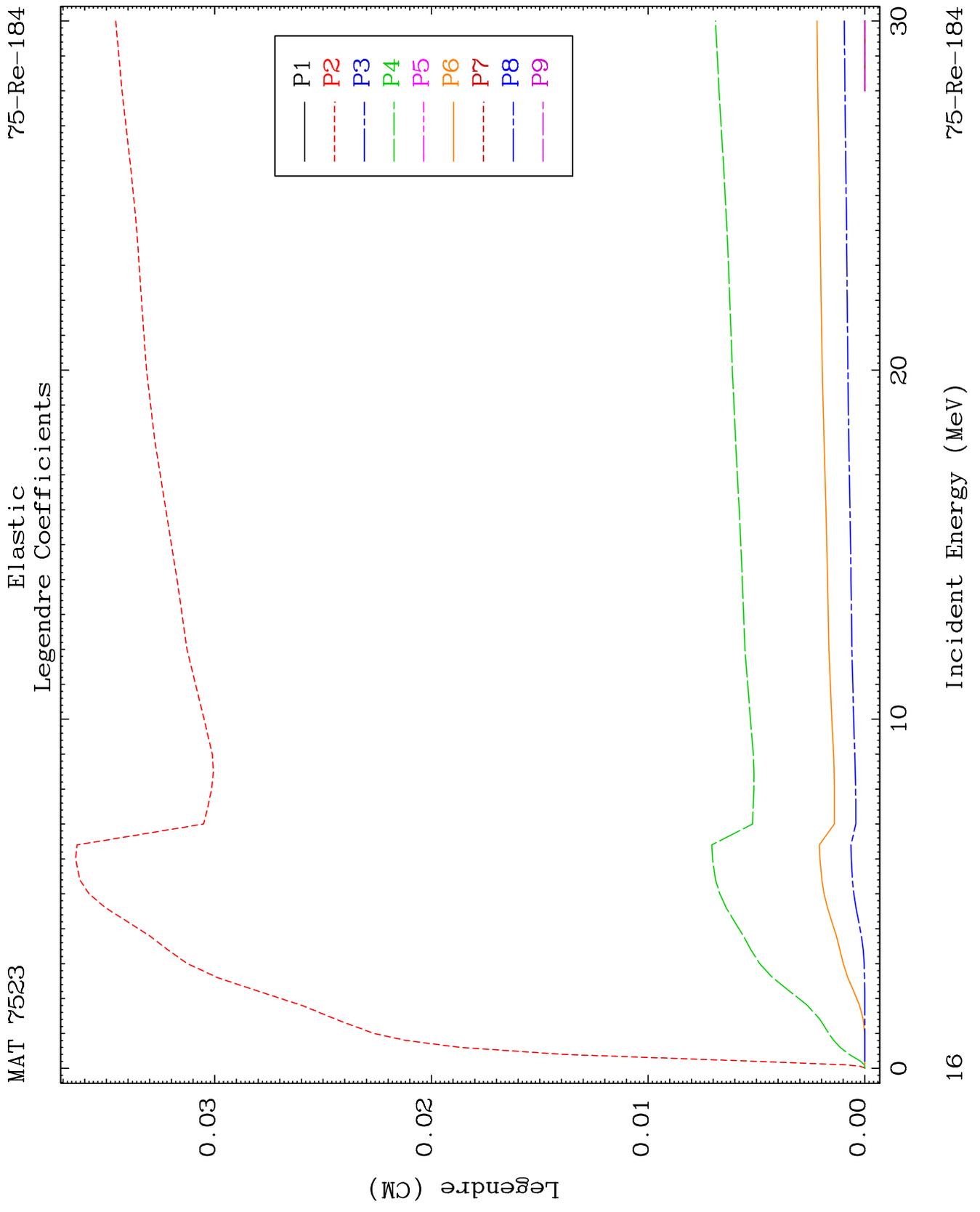
(n,  $\alpha$ ) Levels  
293 Kelvin Cross Sections

75-Re-184



(n,  $\alpha$ )

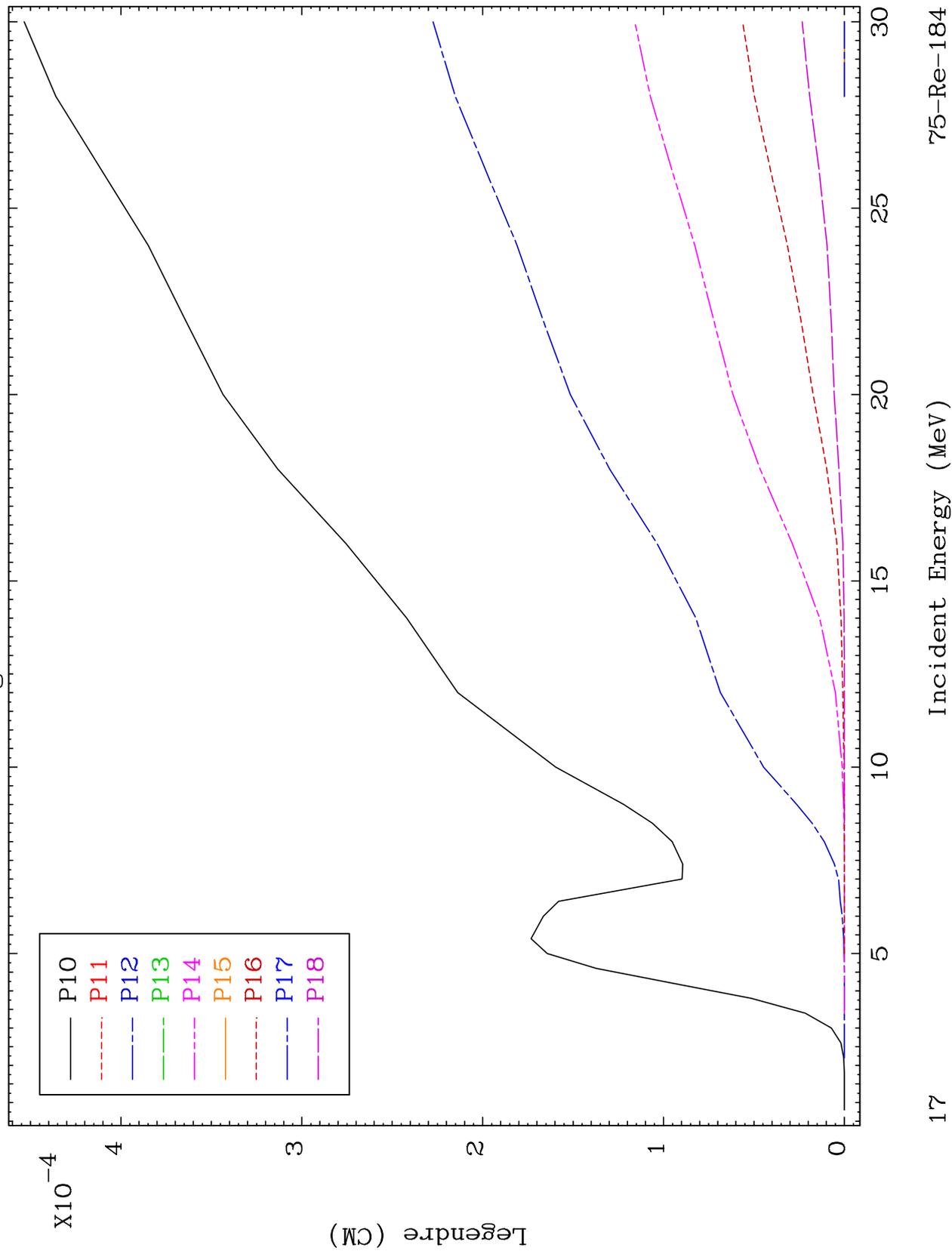




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Elastic Legendre Coefficients

75-Re-184

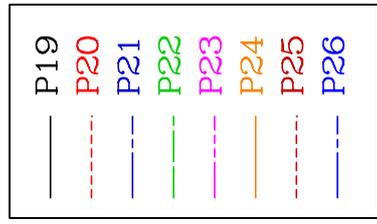


- P10
- P11
- P12
- P13
- P14
- P15
- P16
- P17
- P18

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Elastic  
Legendre Coefficients

75-Re-184



$\times 10^{-6}$

Legendre (CM)

6

4

2

0

15

20

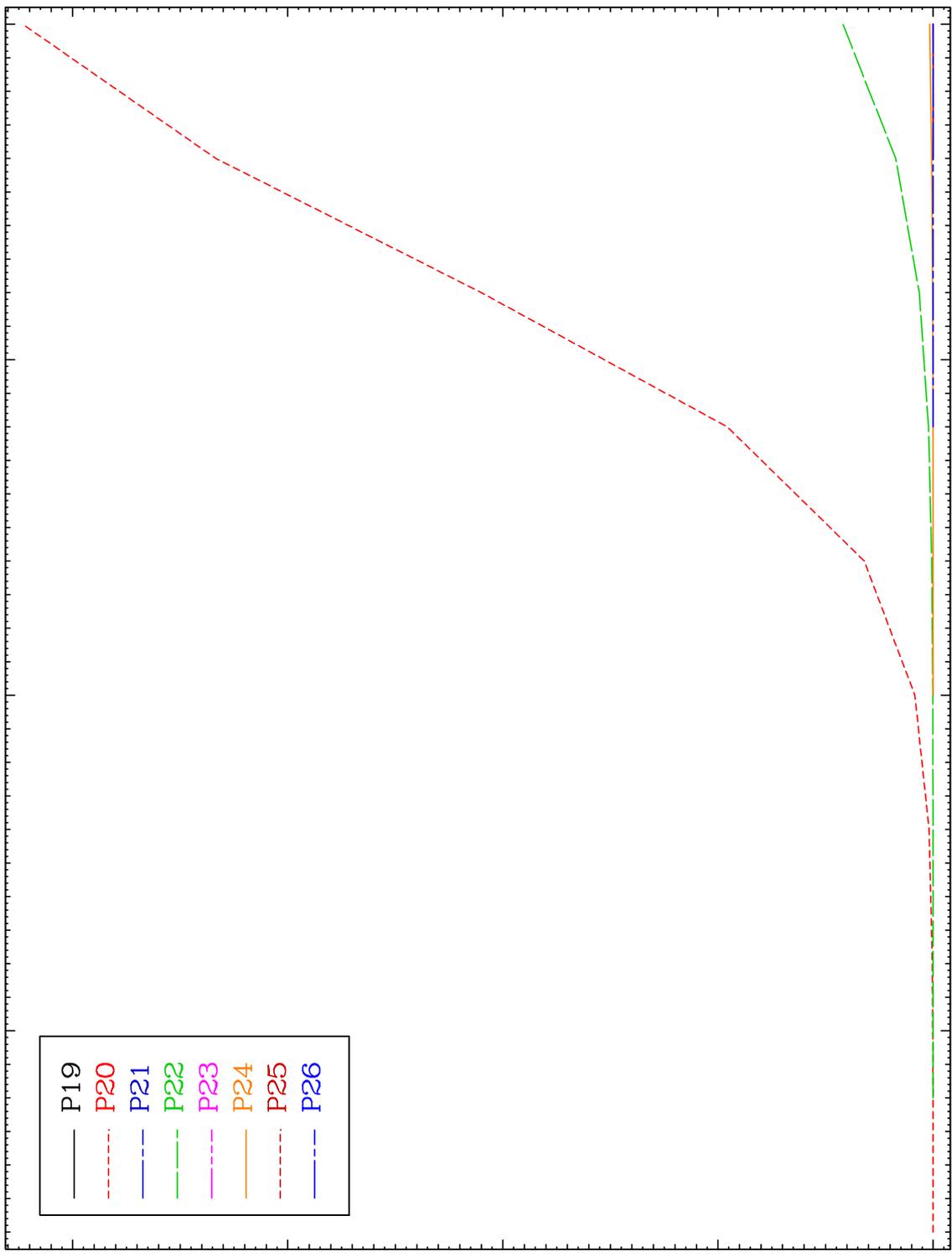
25

30

18

Incident Energy (MeV)

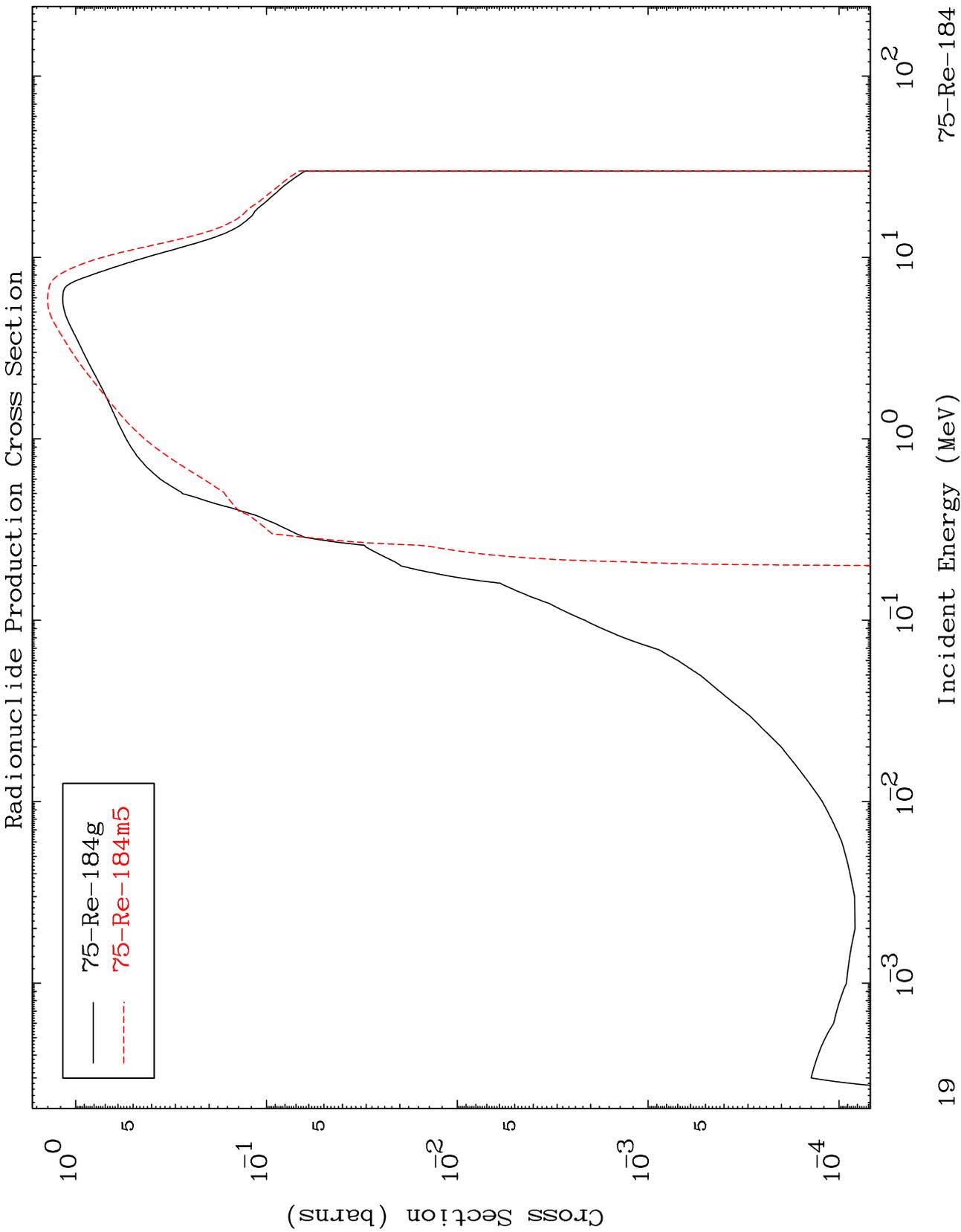
75-Re-184



MAT 7523

75-Re-184

Inelastic  
Radionuclide Production Cross Section



19

75-Re-184

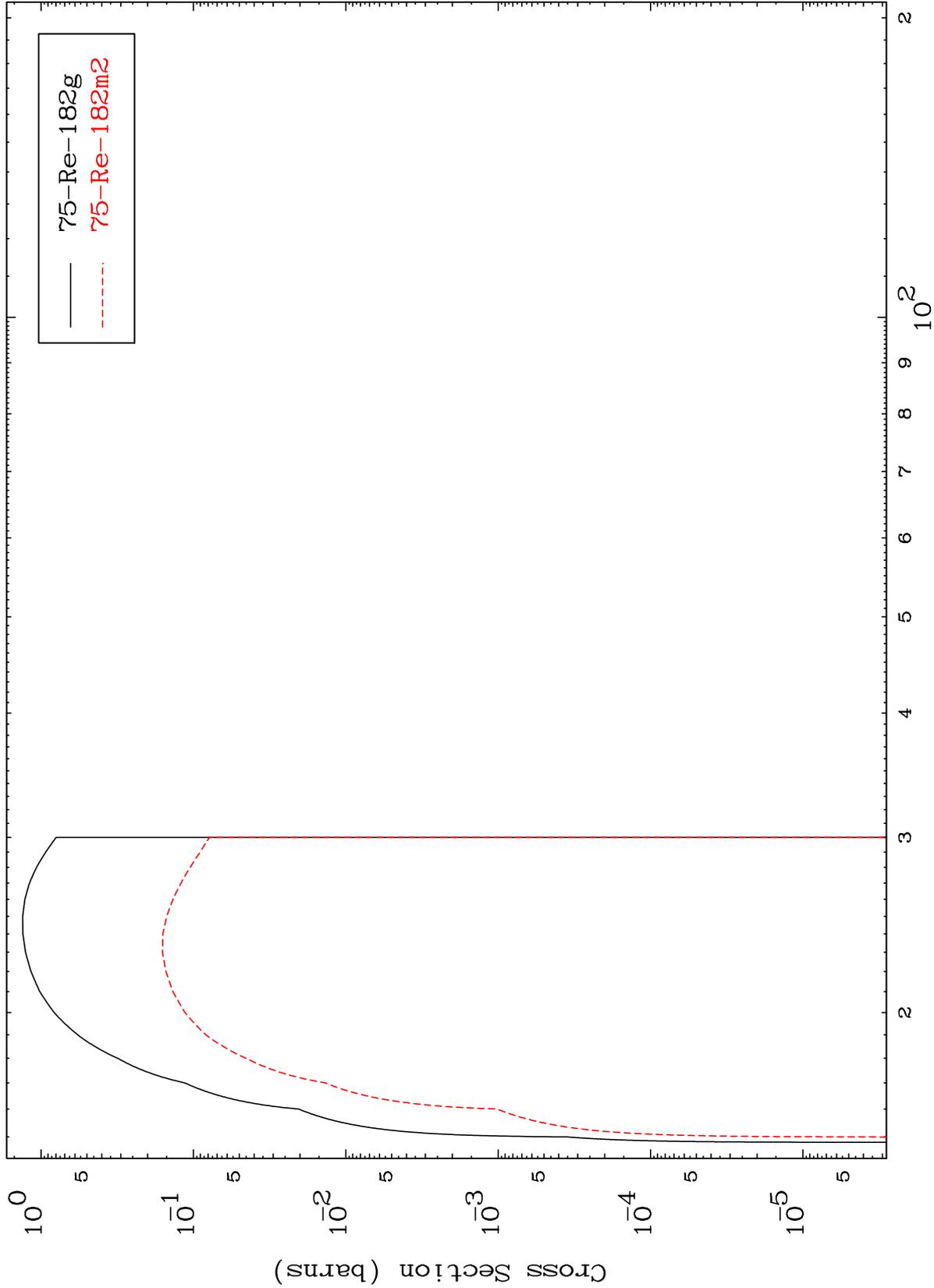
Incident Energy (MeV)

MAT 7523

(n,3n)

<sup>75</sup>Re-<sup>184</sup>

Radionuclide Production Cross Section



20

Incident Energy (MeV)

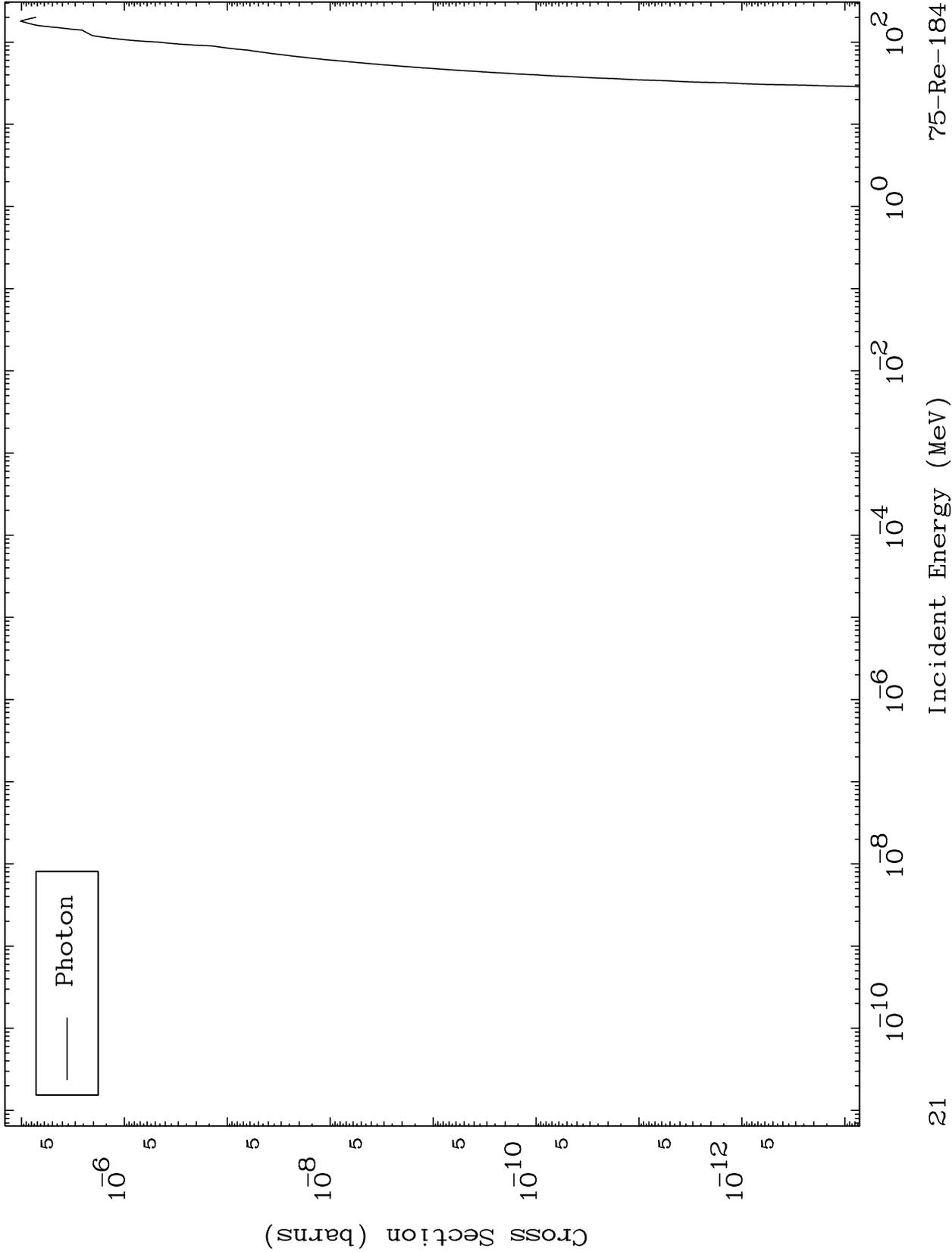
<sup>75</sup>Re-<sup>184</sup>

MAT 7523

Fission

75-Re-184

Radionuclide Production Cross Section

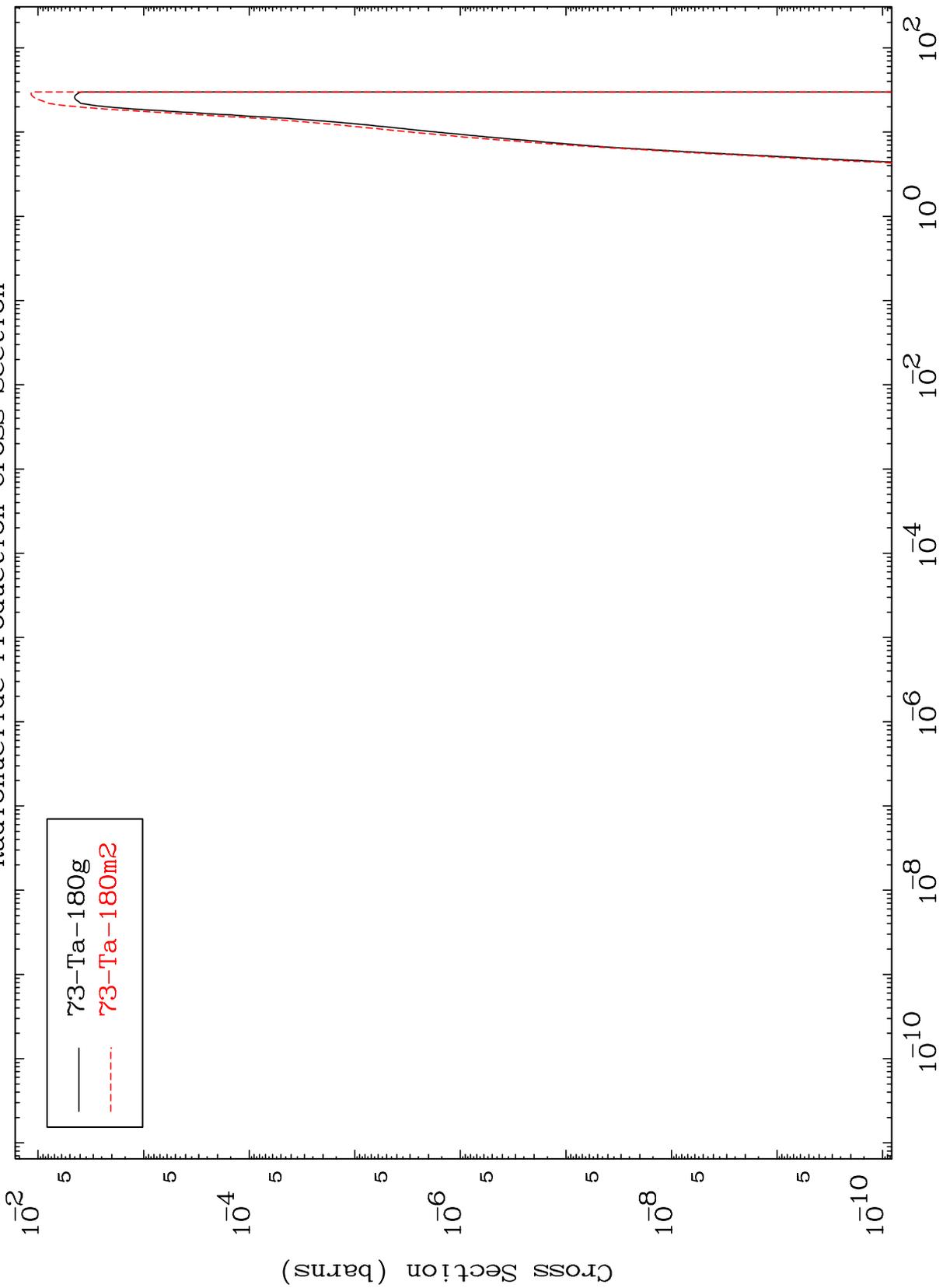


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$(n, n') \alpha$

<sup>75</sup>Re-184

Radionuclide Production Cross Section

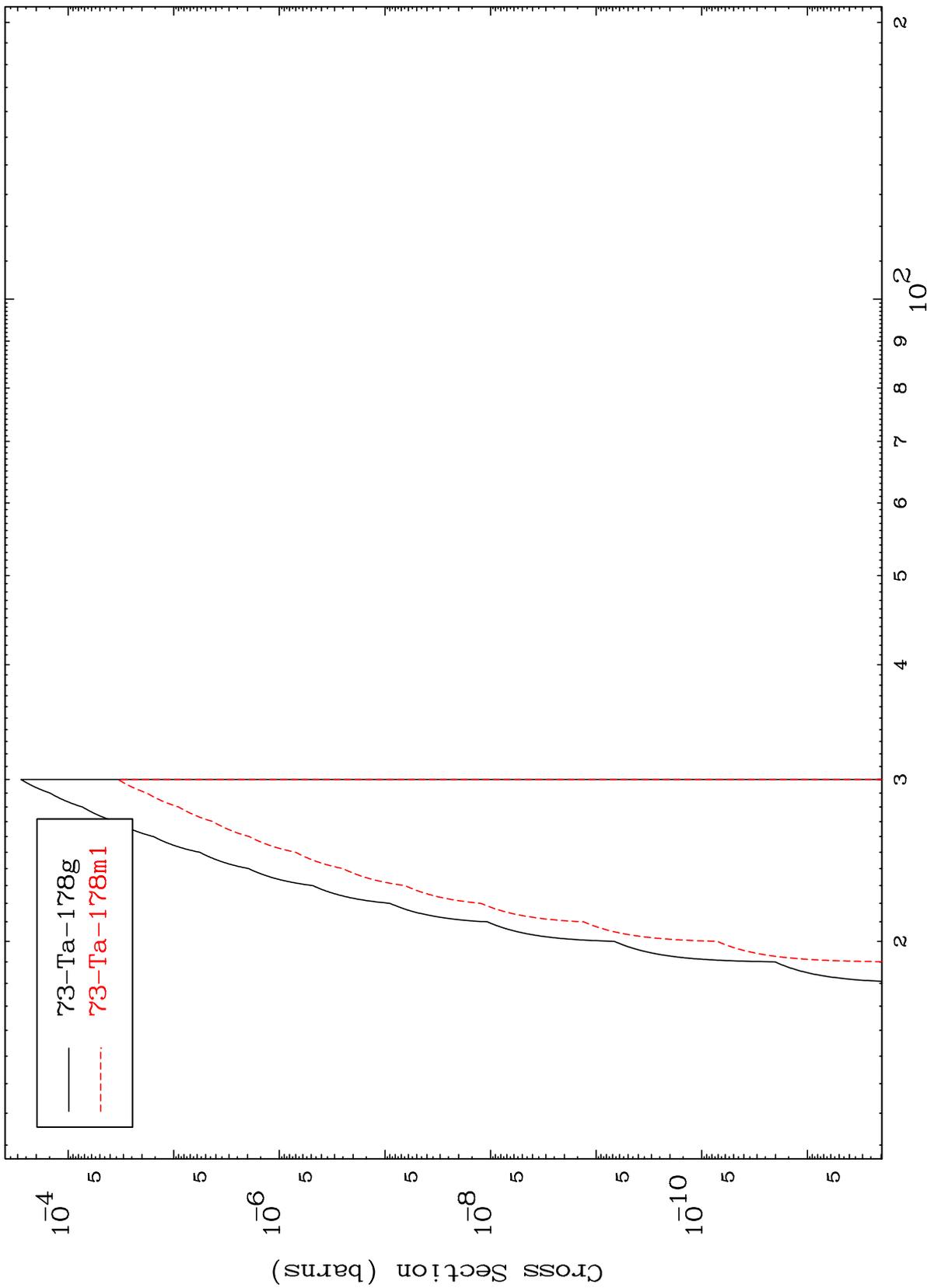


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(n,3n)  $\alpha$

75-Re-184

Radionuclide Production Cross Section

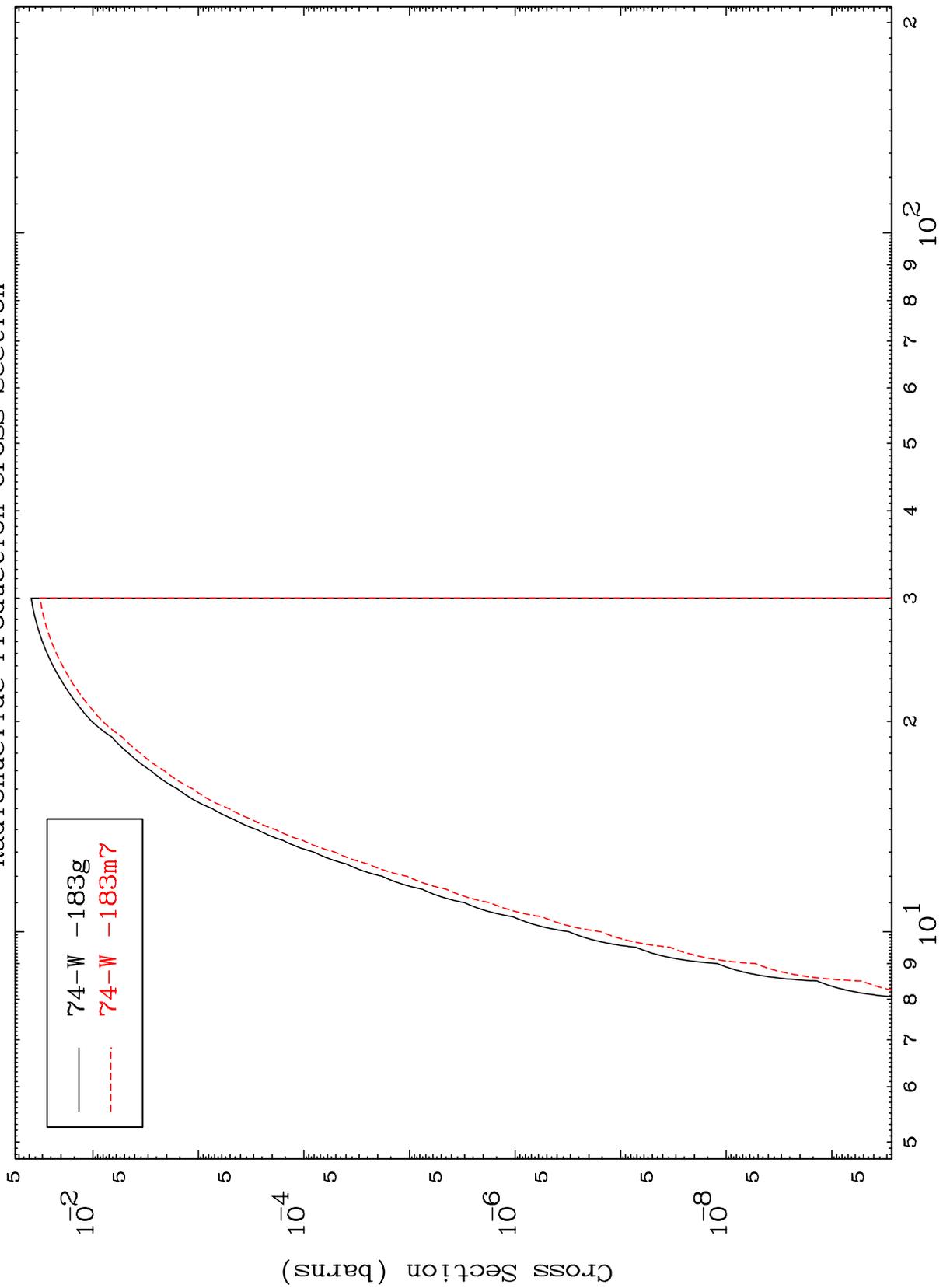


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(n,n') p

75-Re-184

Radionuclide Production Cross Section



Incident Energy (MeV)

75-Re-184

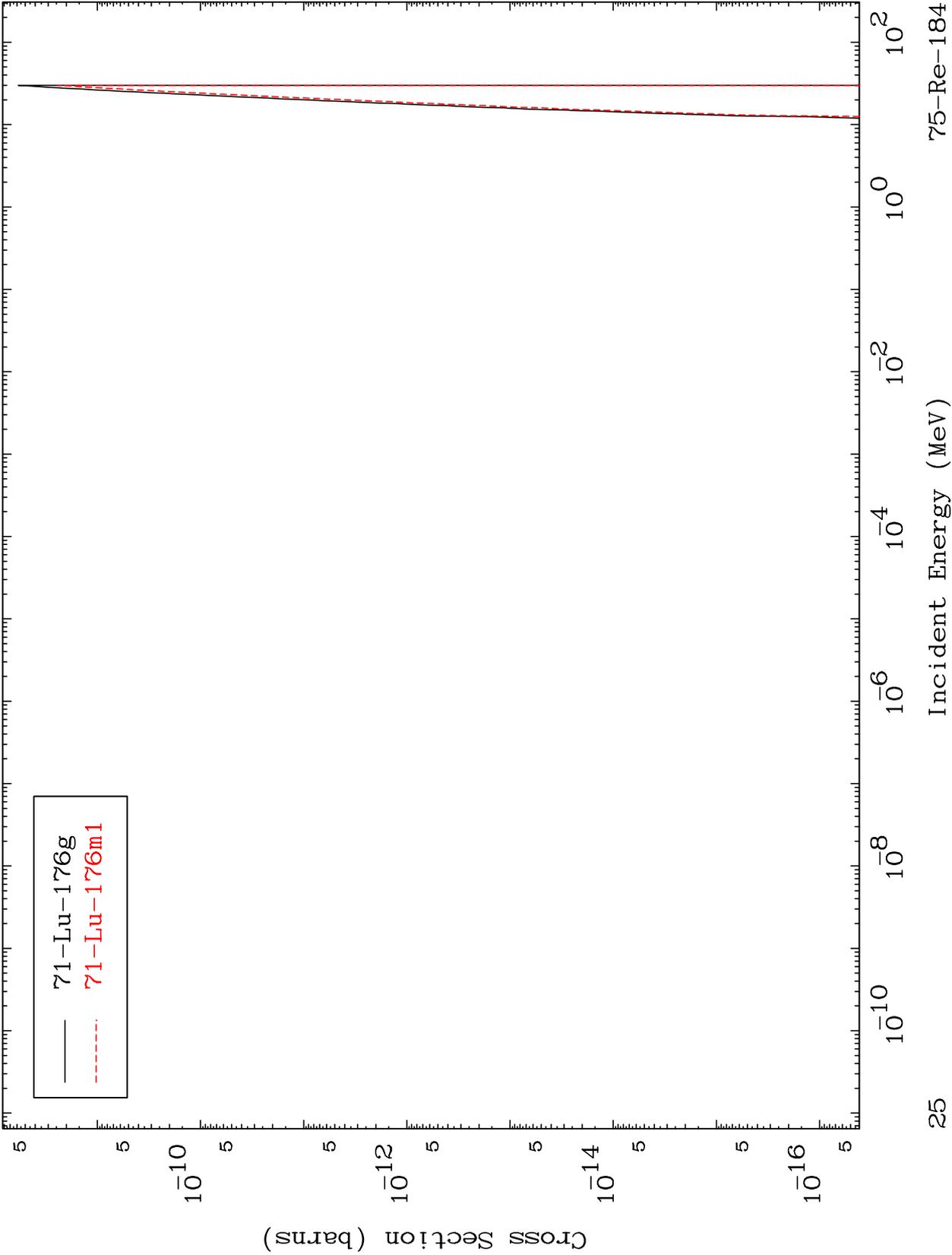
24

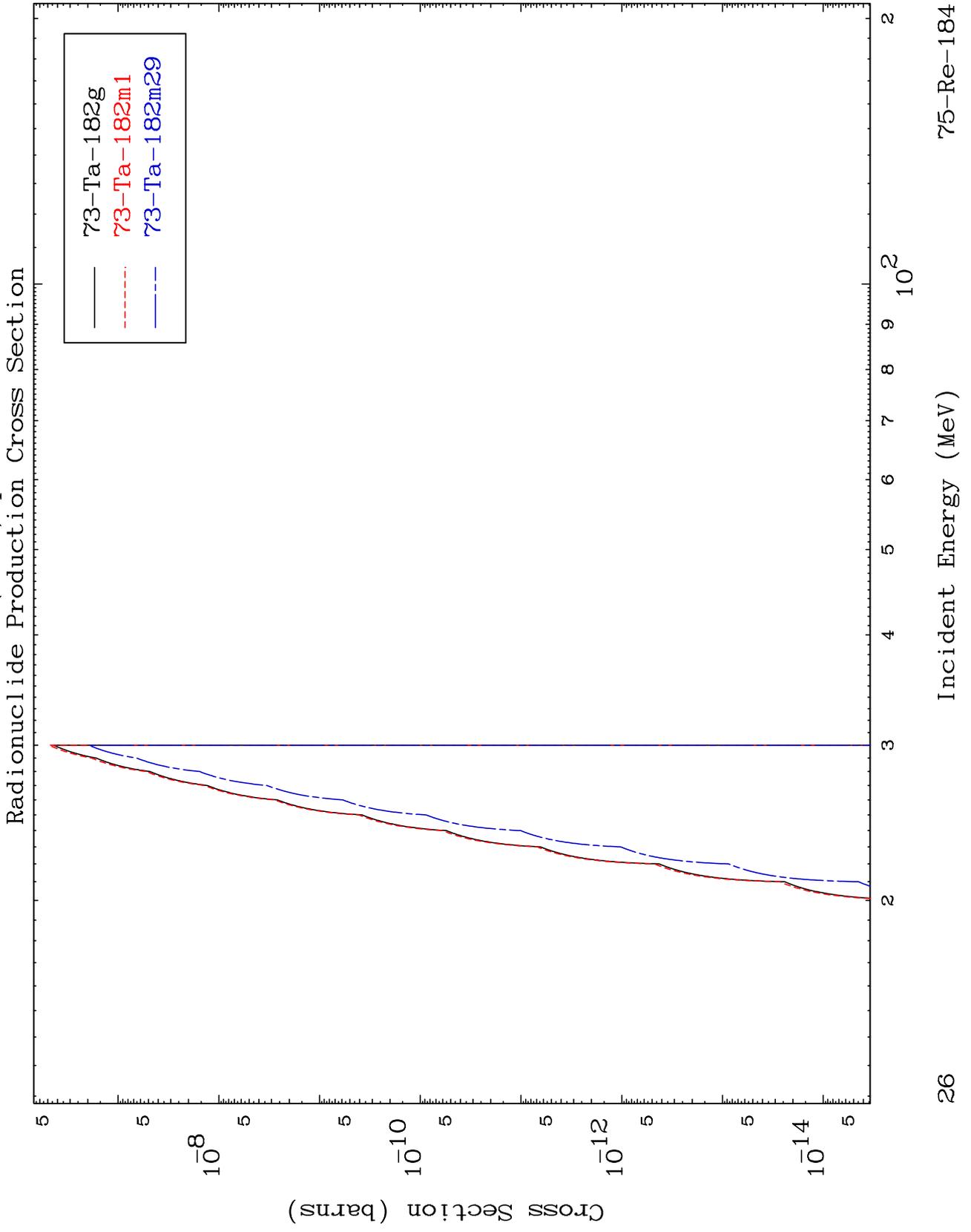
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(n,n') 2α

75-Re-184

Radionuclide Production Cross Section



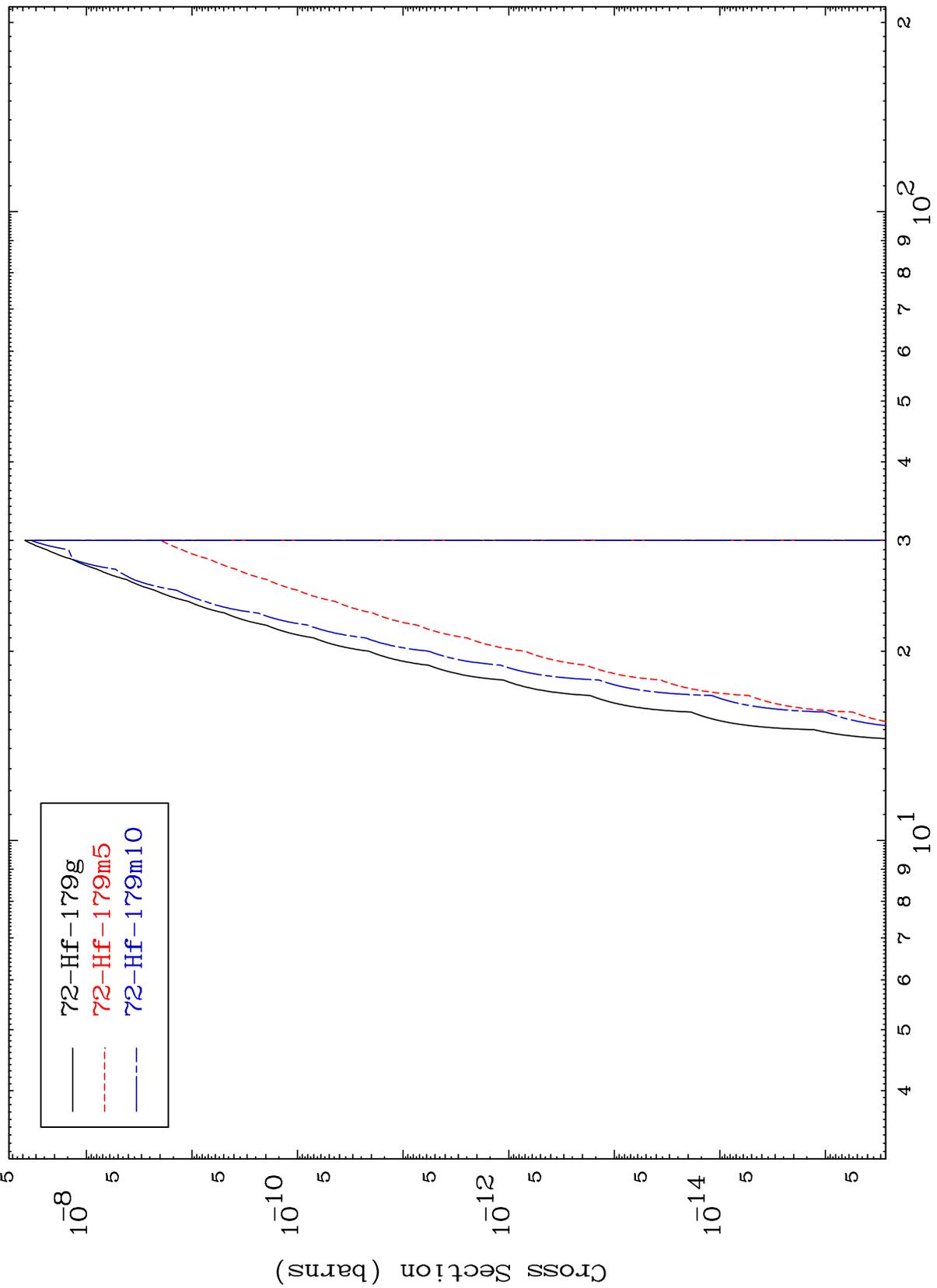


MAT 7523

(n,n') p  $\alpha$

75-Re-184

Radionuclide Production Cross Section



27

Incident Energy (MeV)

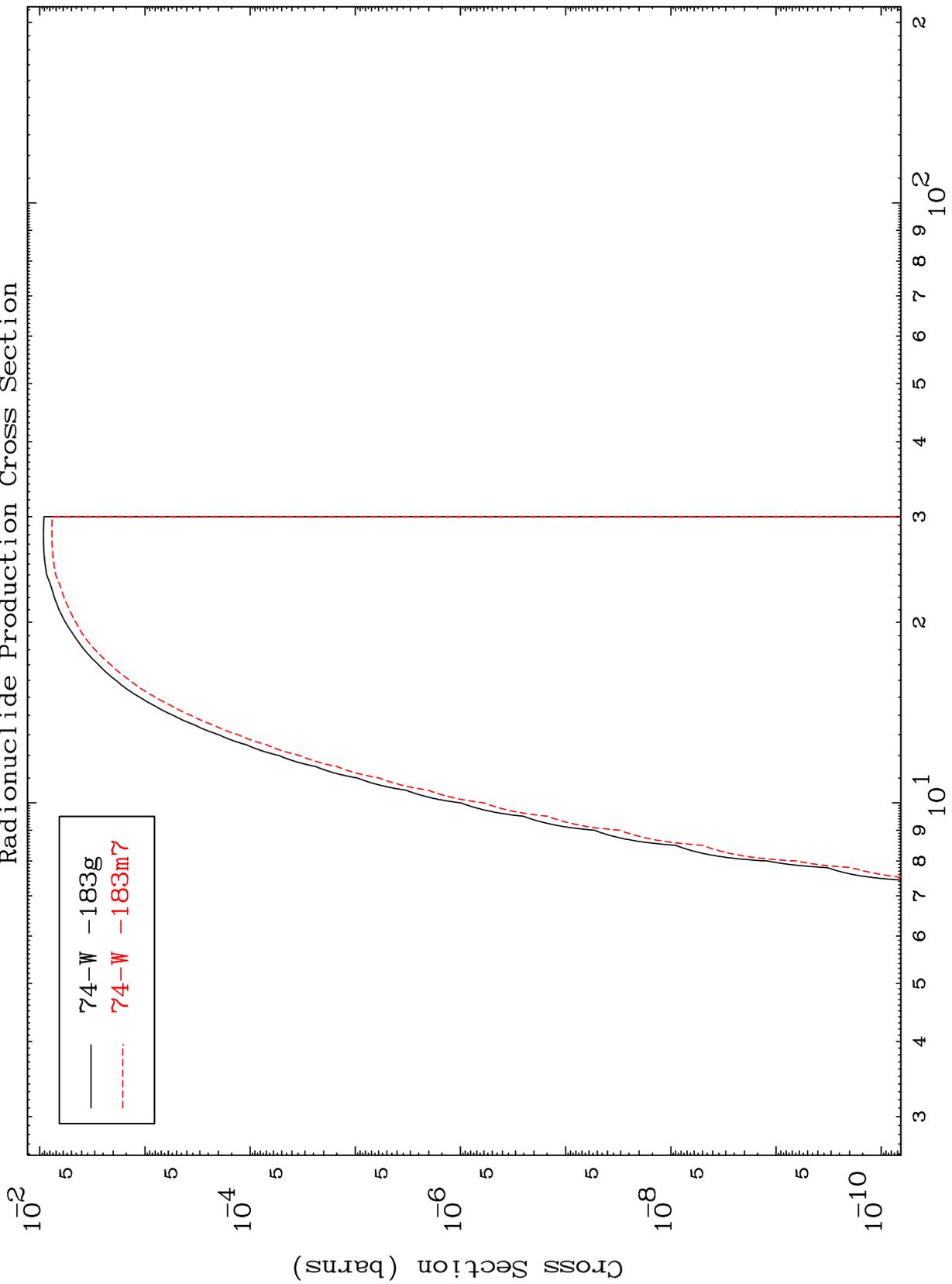
75-Re-184

MAT 7523

(n,d)

75-Re-184

Radionuclide Production Cross Section



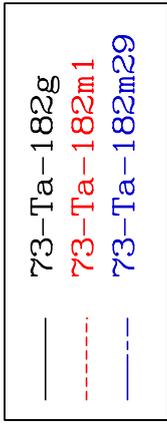
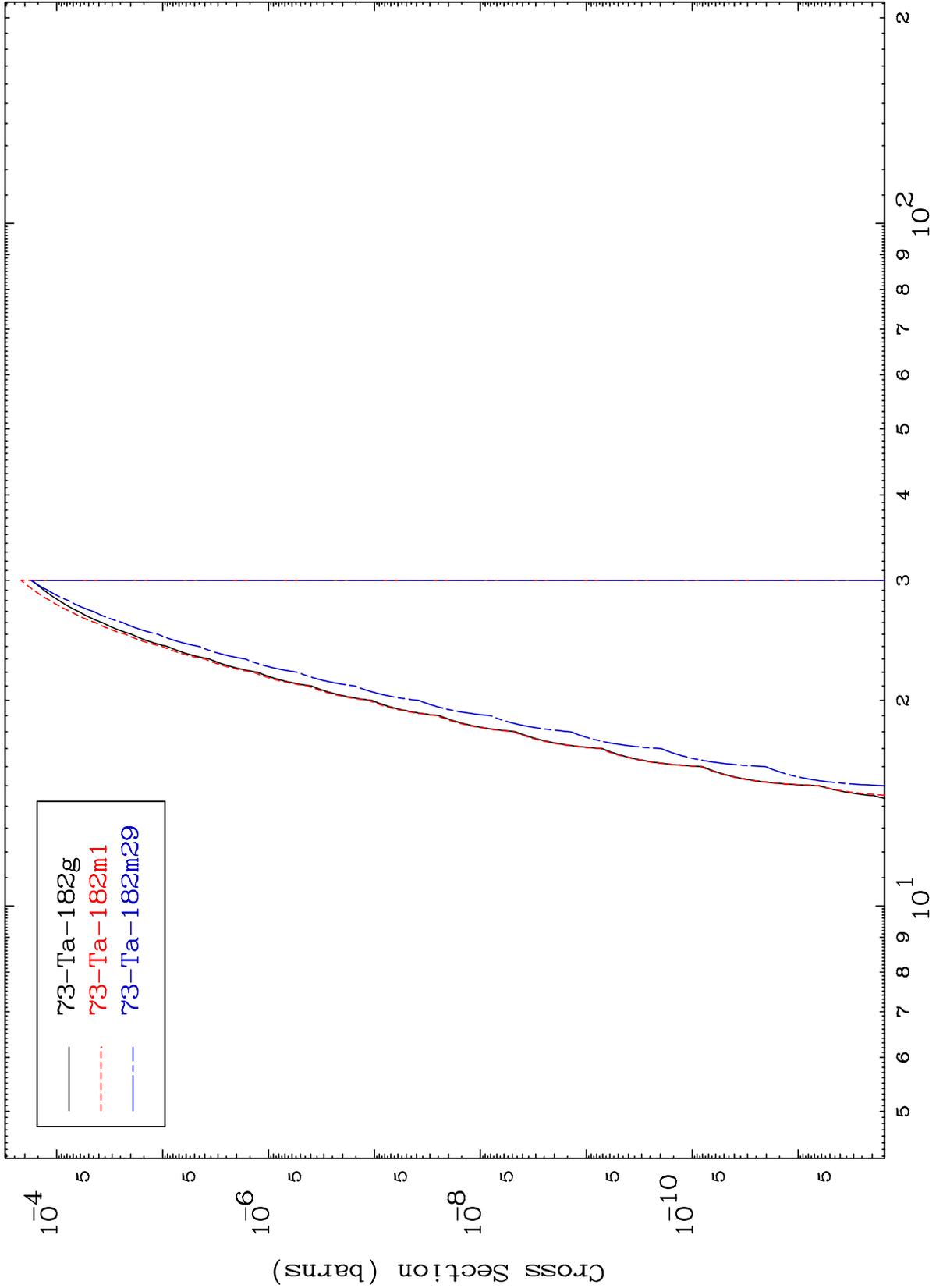
74-W -183g  
74-W -183m7

Incident Energy (MeV)

75-Re-184

28

Radionuclide Production Cross Section (n,He-3)

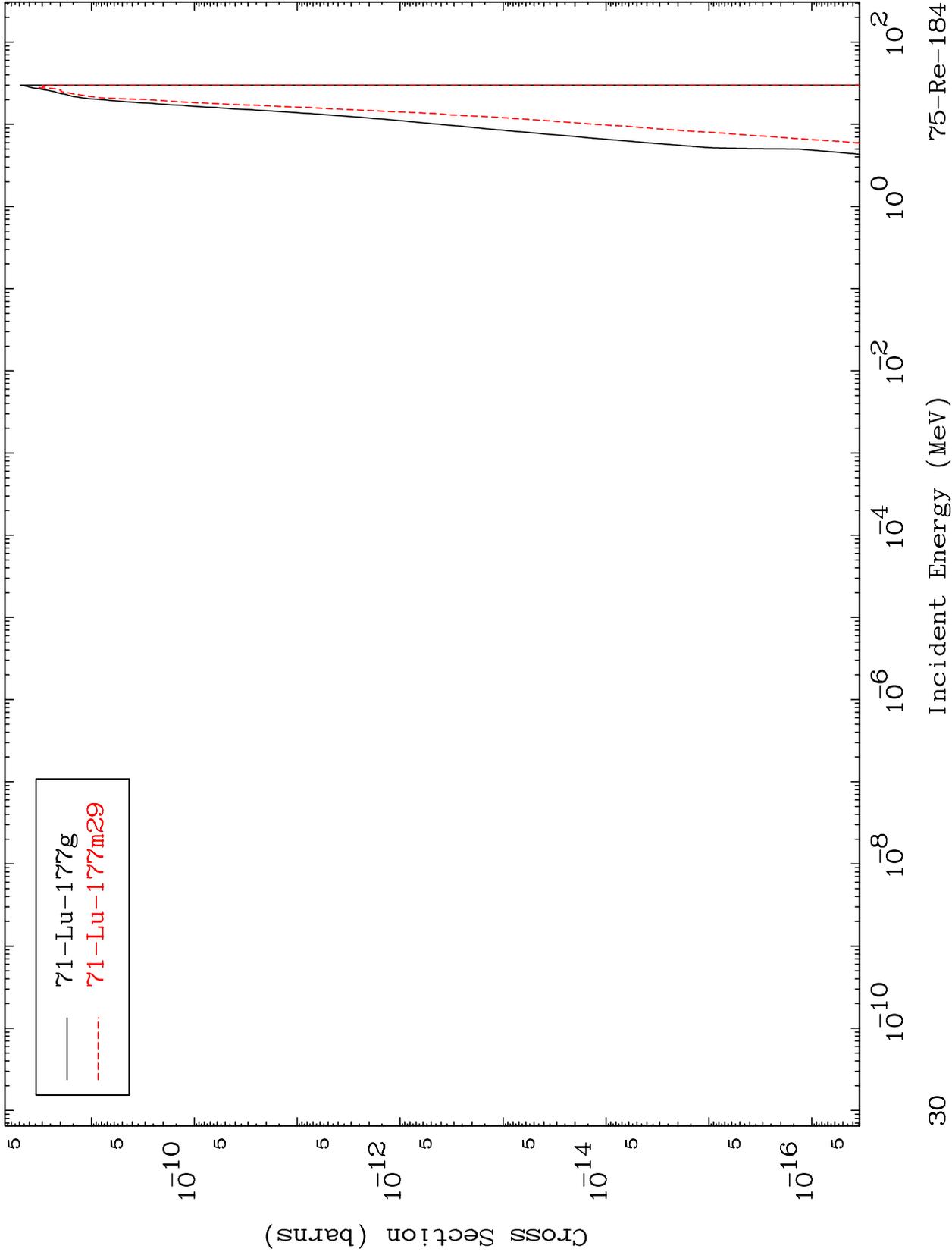


MAT 7523

(n,2α)

75-Re-184

Radionuclide Production Cross Section

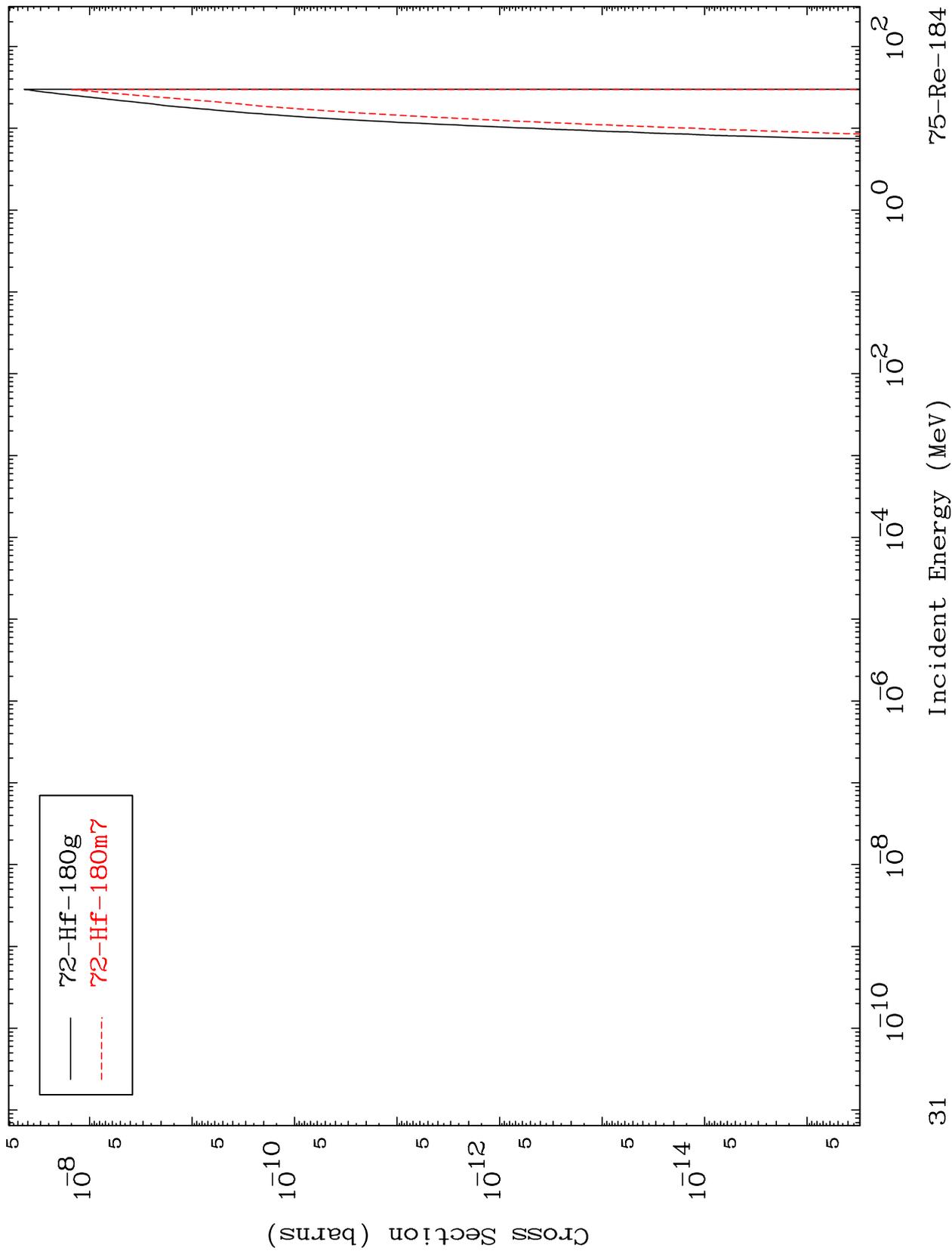


MAT 7523

(n,p)  $\alpha$

<sup>75</sup>Re-184

Radionuclide Production Cross Section



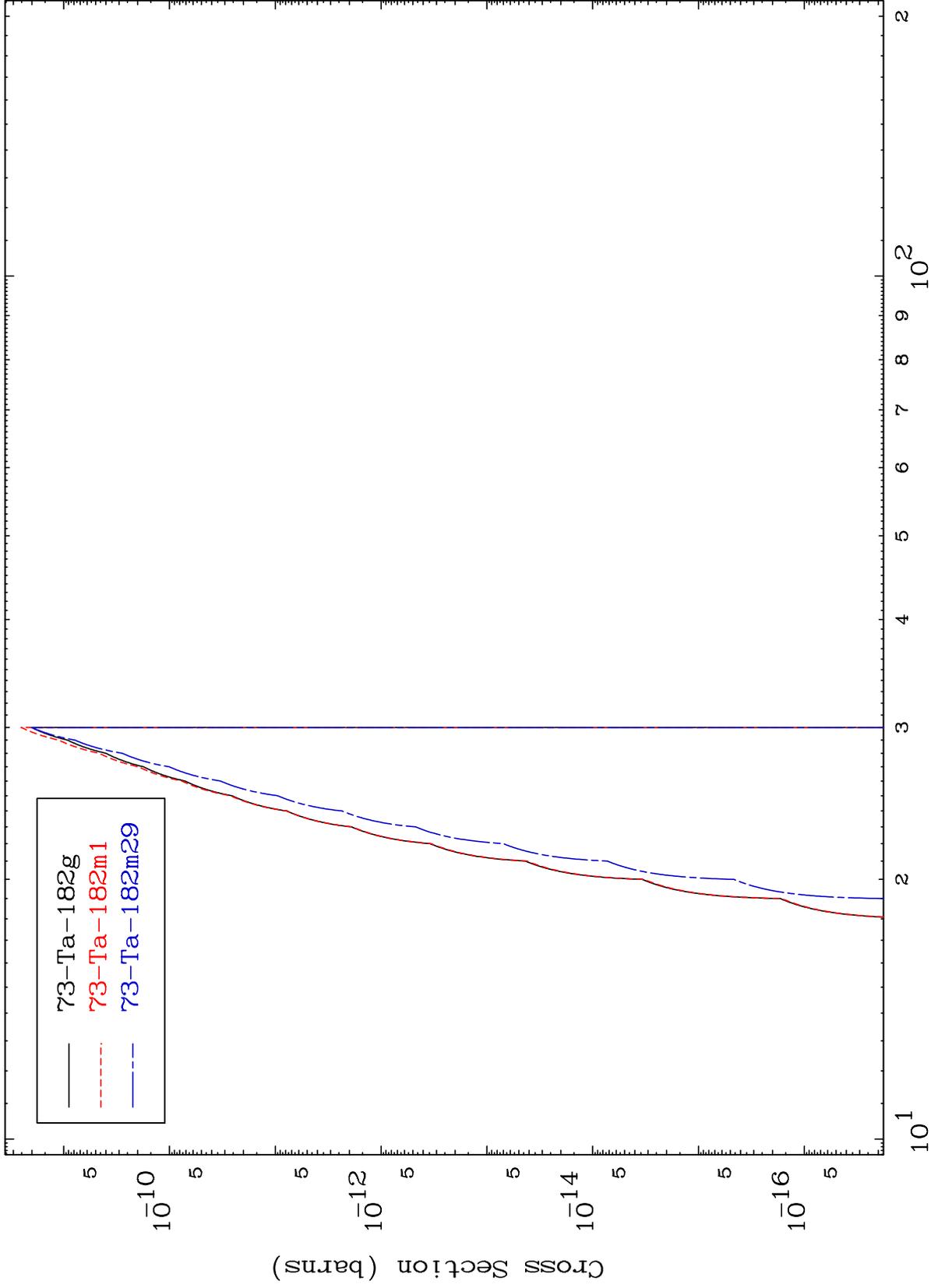
— 72-Hf-180g  
- - - 72-Hf-180m7

MAT 7523

(n,p) d

75-Re-184

Radionuclide Production Cross Section



Incident Energy (MeV)

75-Re-184

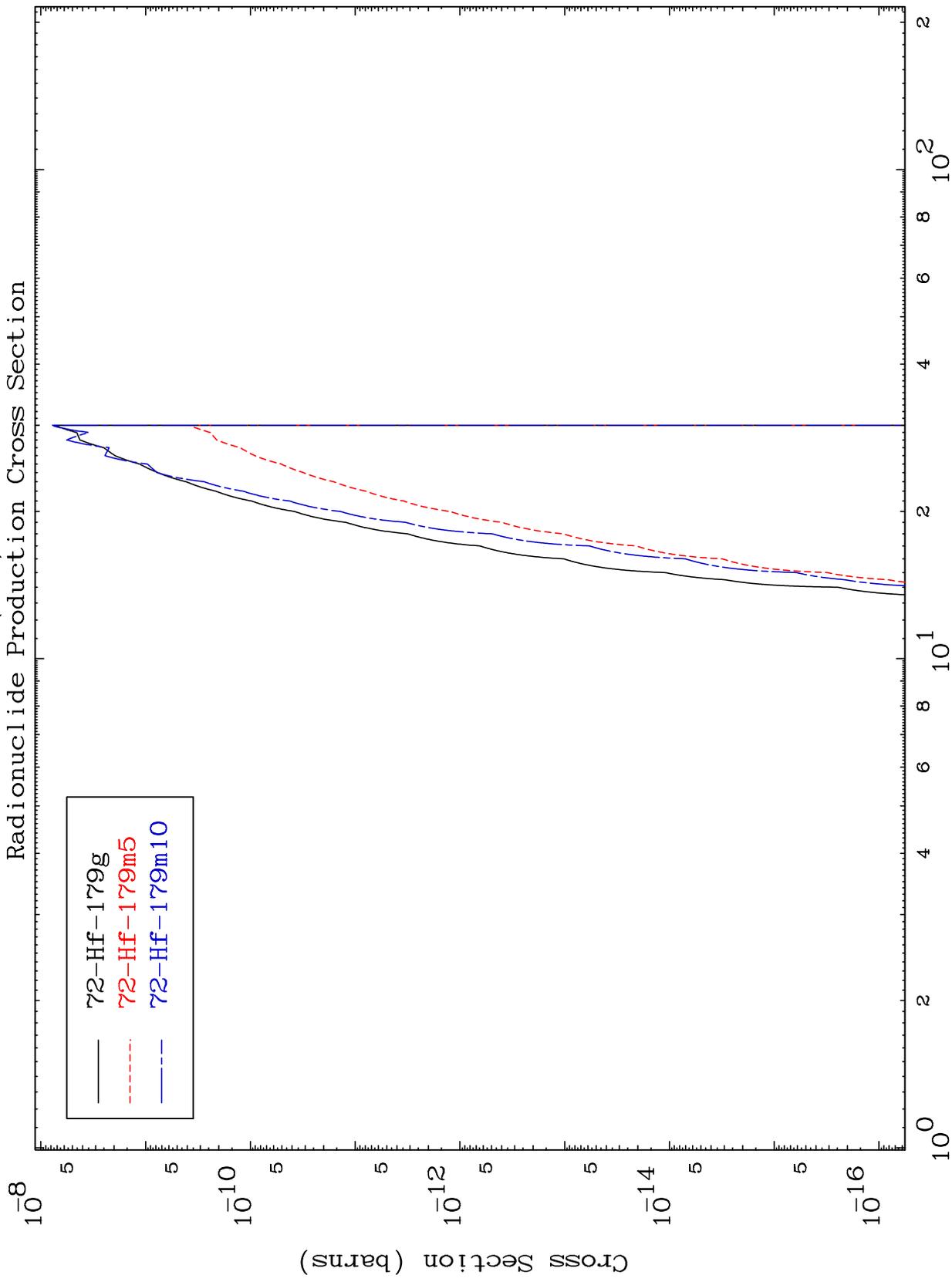
32

MAT 7523

(n,d)  $\alpha$

75-Re-184

Radionuclide Production Cross Section



Incident Energy (MeV)

75-Re-184

33