

Program EVALPLOT
(Version 2015-2)

by

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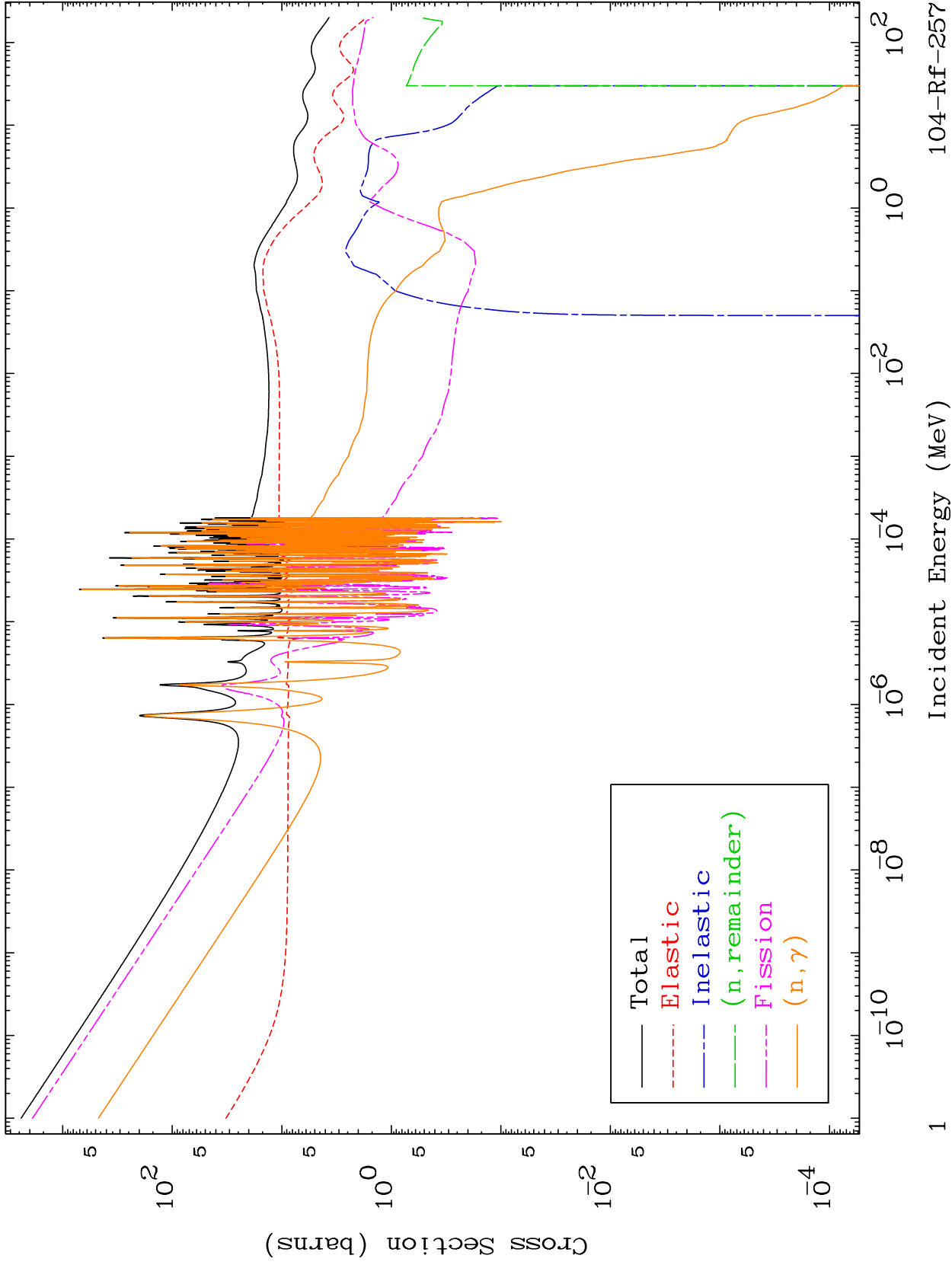
Press Mouse Button to Start

MAT 457

Major

293 Kelvin Cross Sections

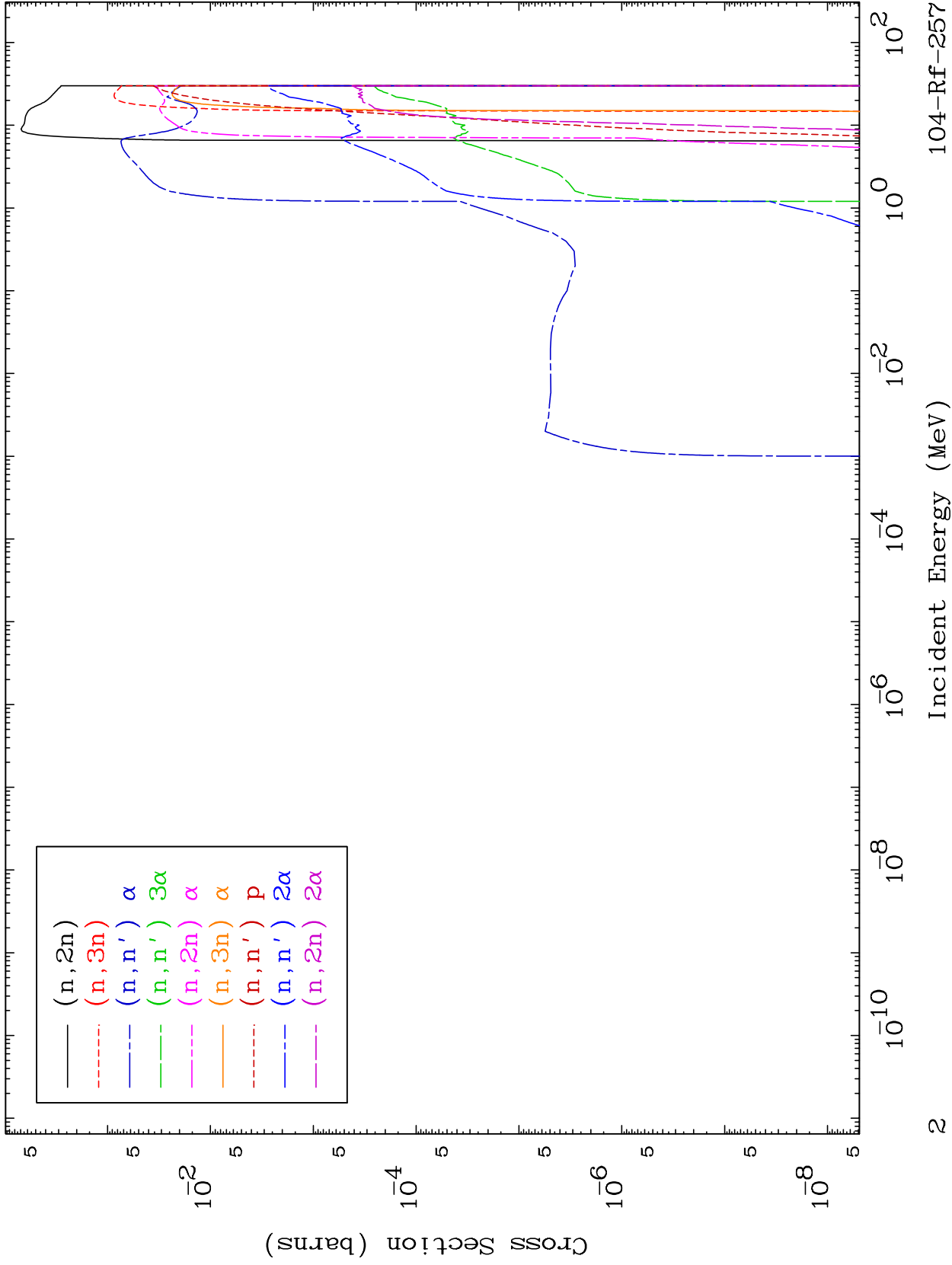
104-Rf-257



MAT 457

Neutron Production
293 Kelvin Cross Sections

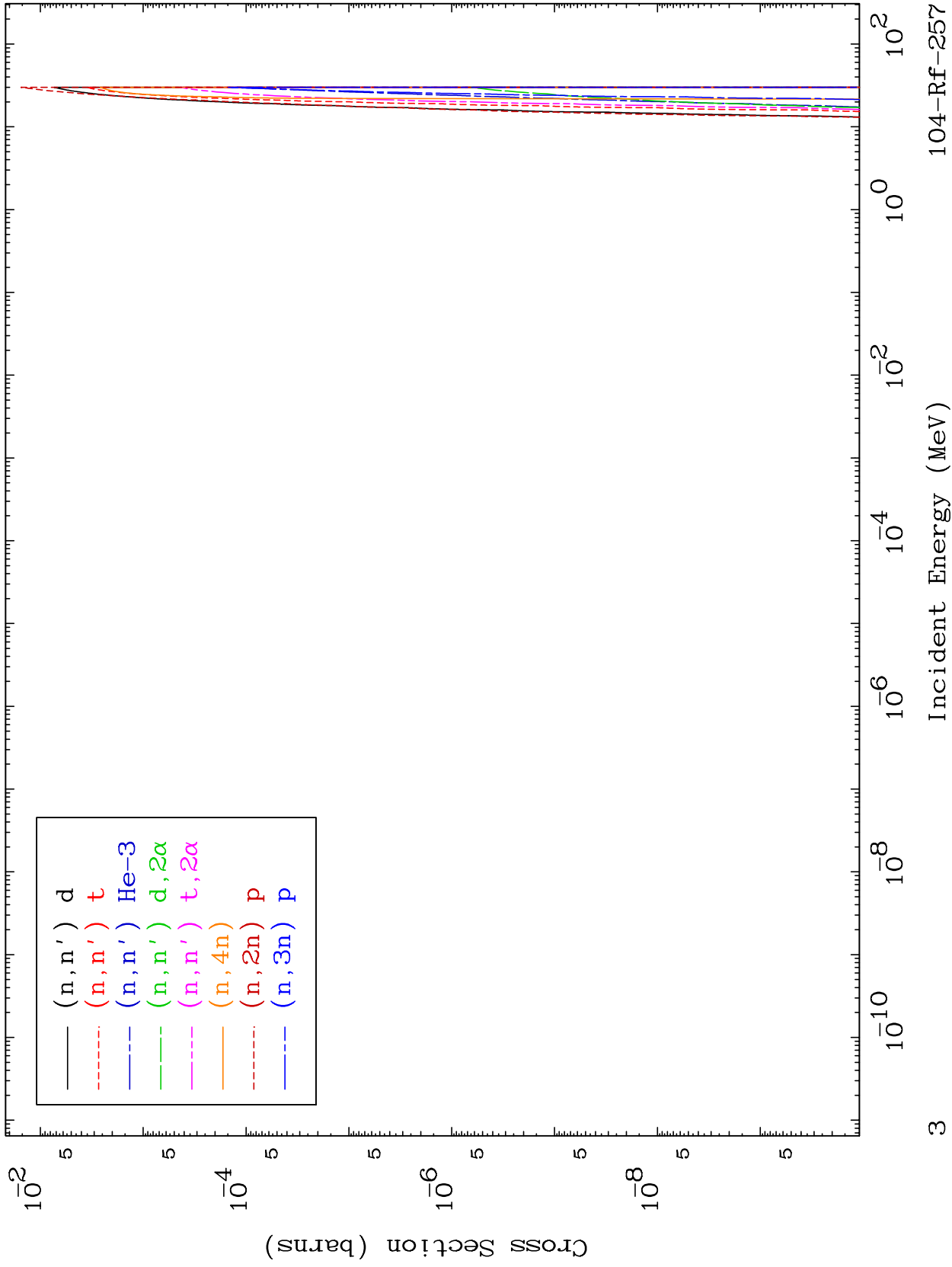
104-Rf-257



MAT 457

Neutron Production
293 Kelvin Cross Sections

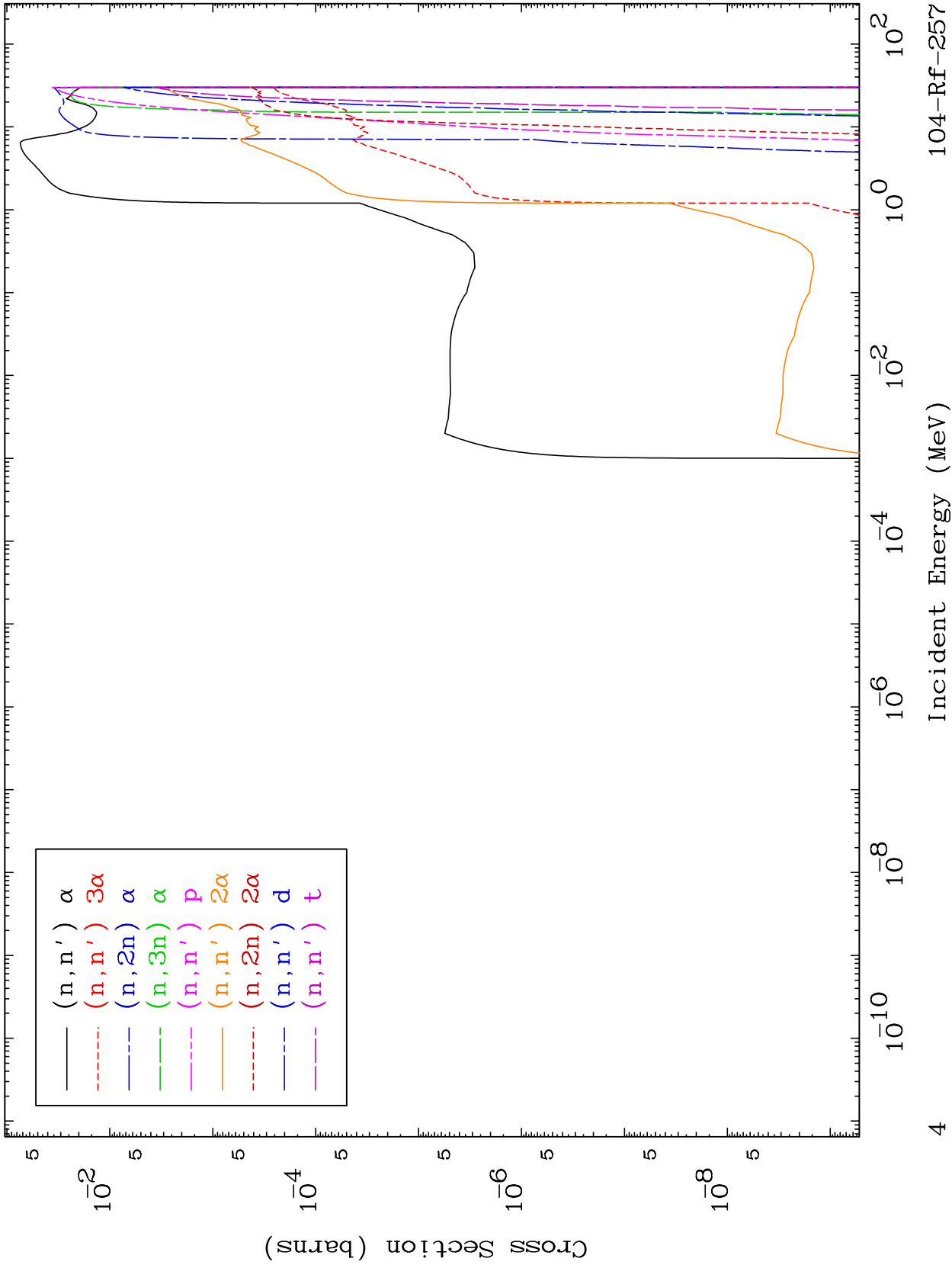
104-Rf-257



MAT 457

Charged Particle
293 Kelvin Cross Sections

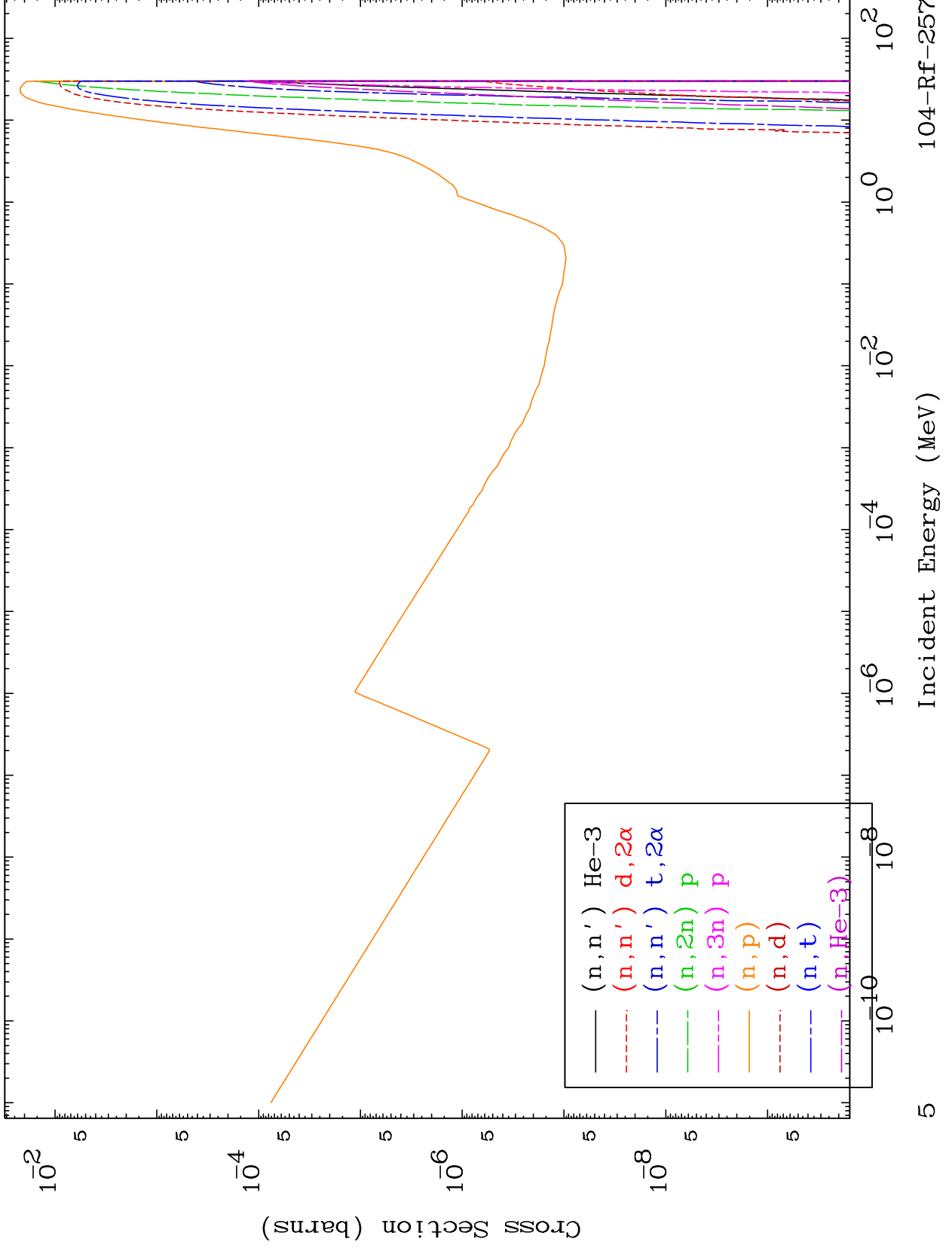
104-Rf-257



MAT 457

Charged Particle
293 Kelvin Cross Sections

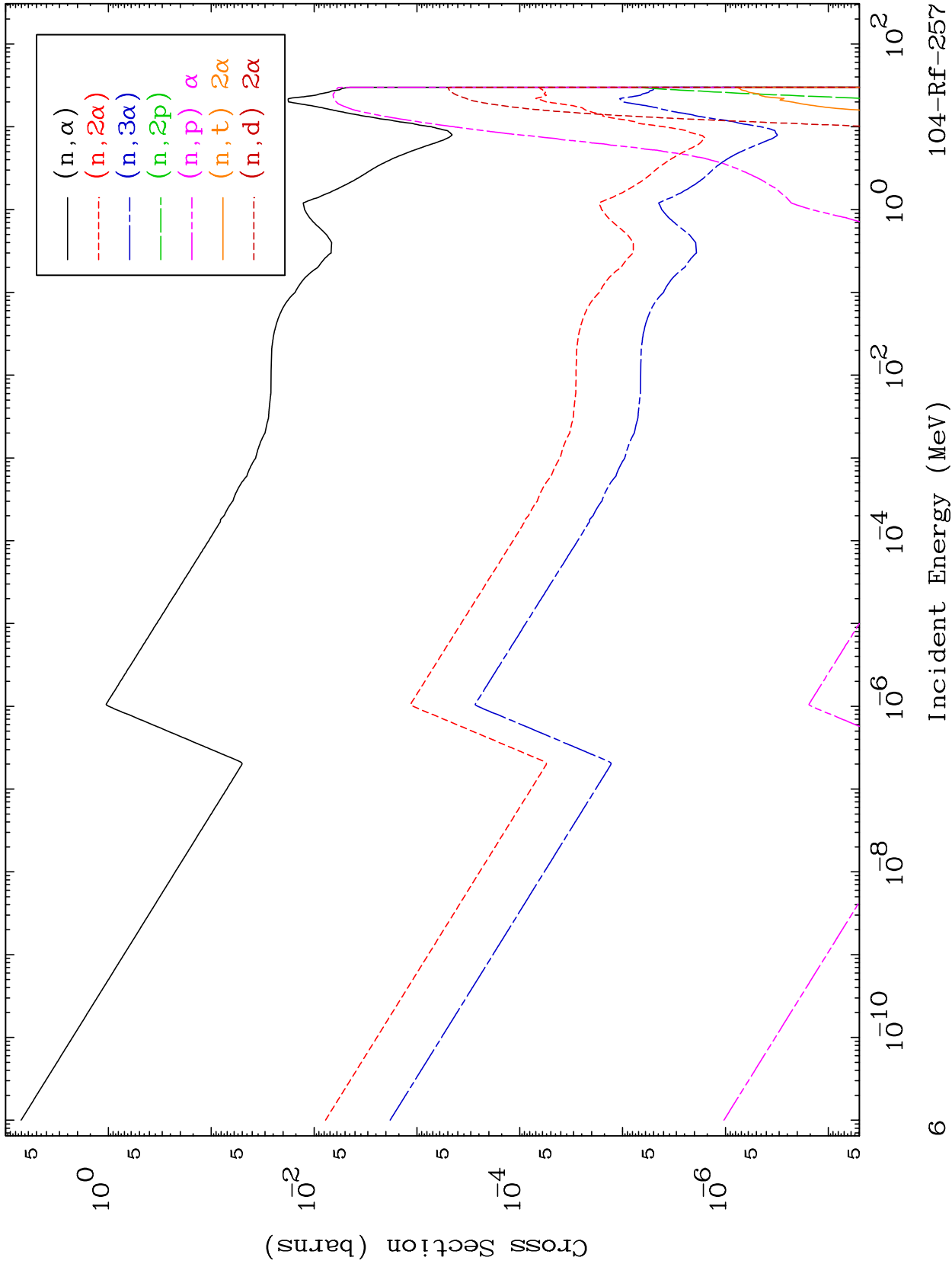
104-Rf-257



MAT 457

Charged Particle
293 Kelvin Cross Sections

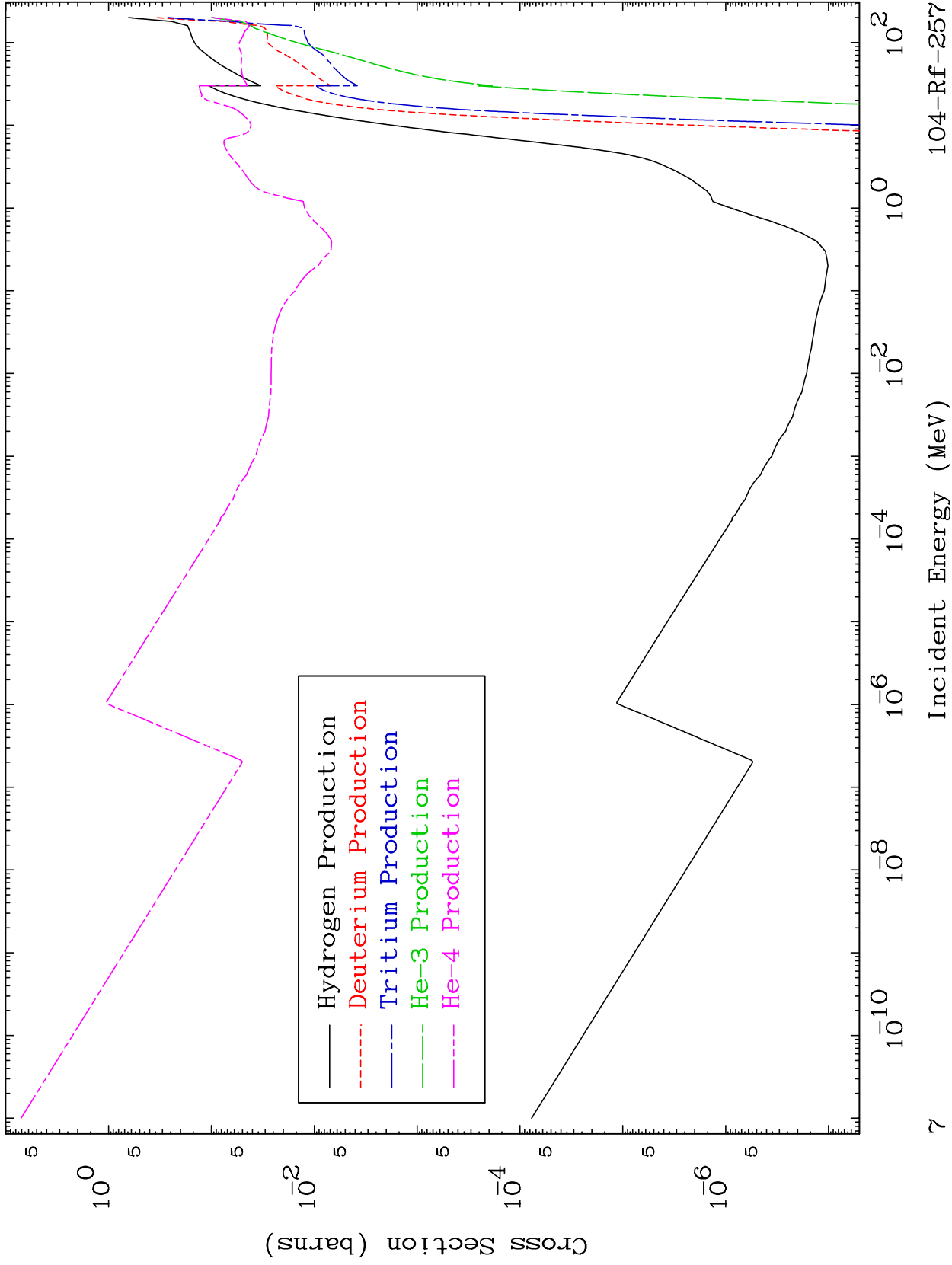
104-Rf-257



MAT 457

Particle Production
293 Kelvin Cross Sections

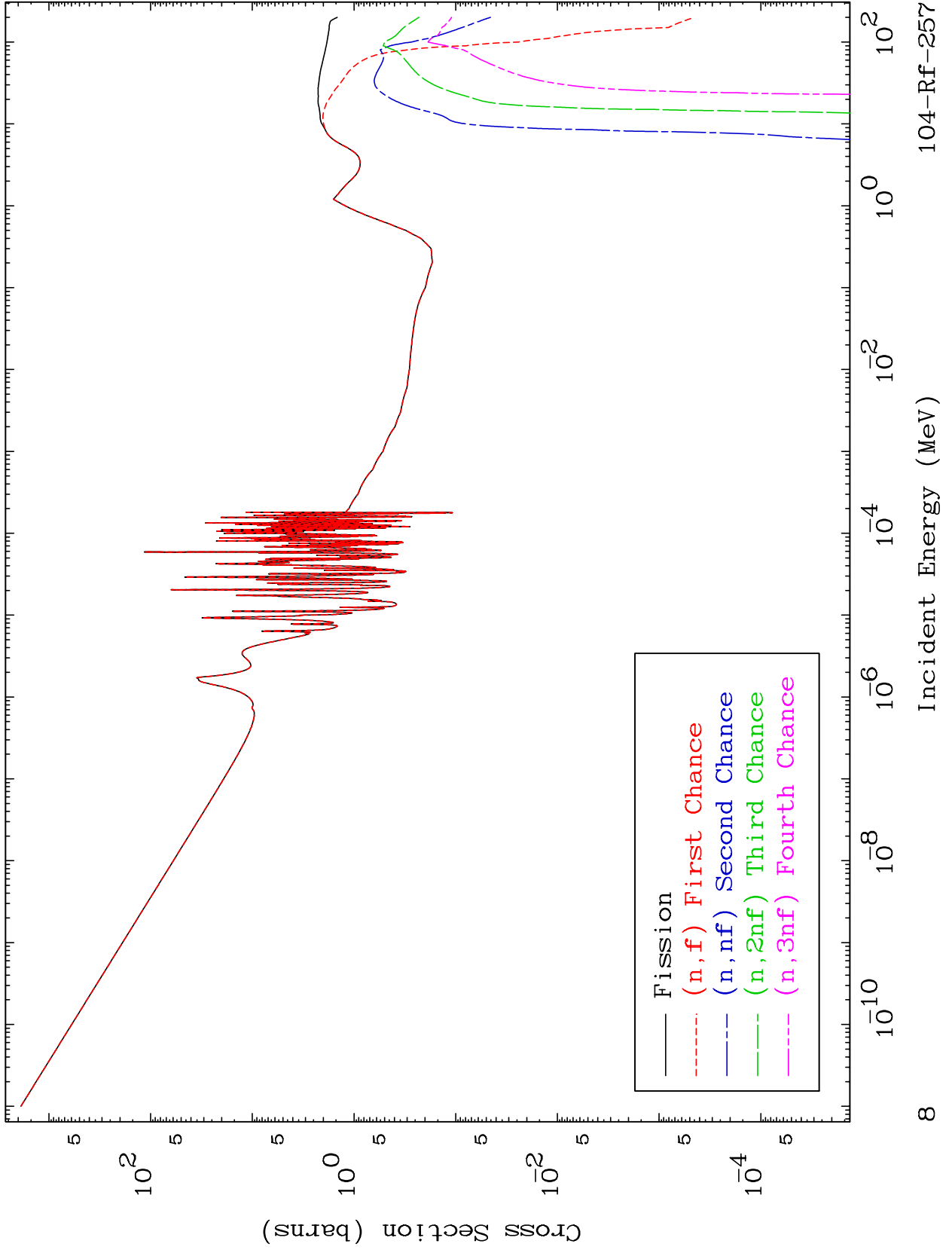
104-Rf-257



MAT 457

Fission
293 Kelvin Cross Sections

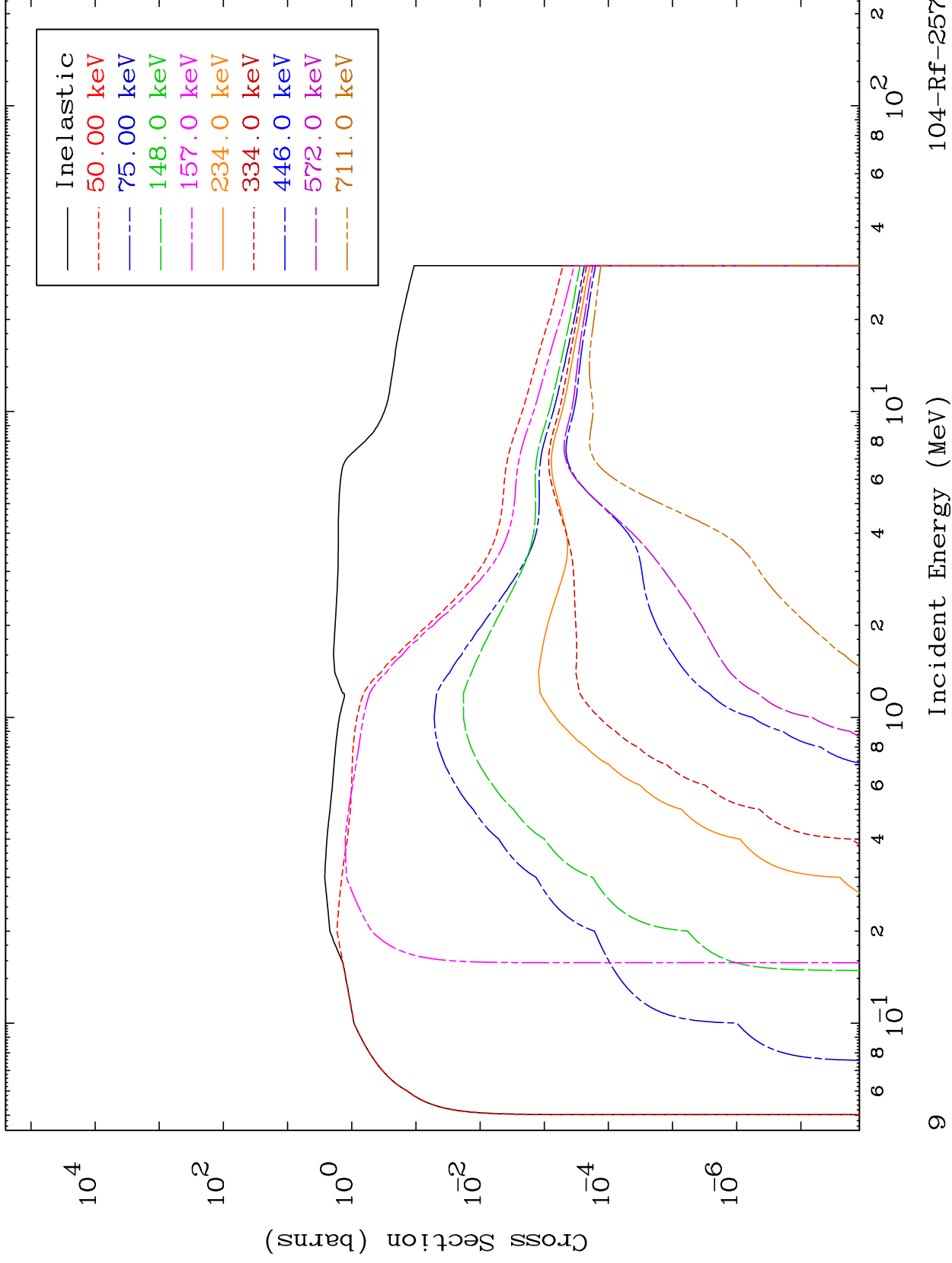
104-Rf-257



MAT 457

(n,n') Level
293 Kelvin Cross Sections

104-Rf-257

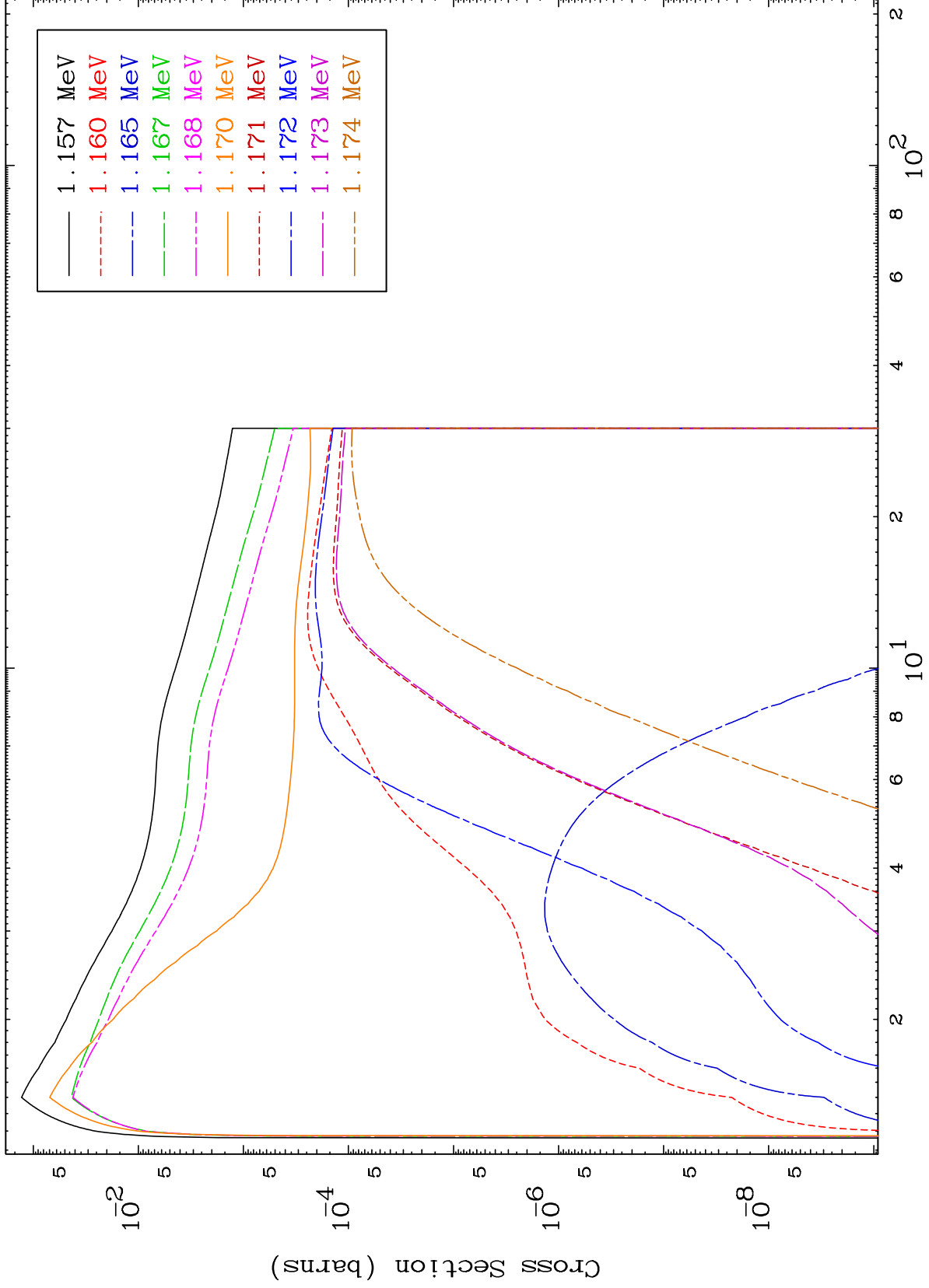


MAT 457

(n,n') Level

104-Rf-257

293 Kelvin Cross Sections



10

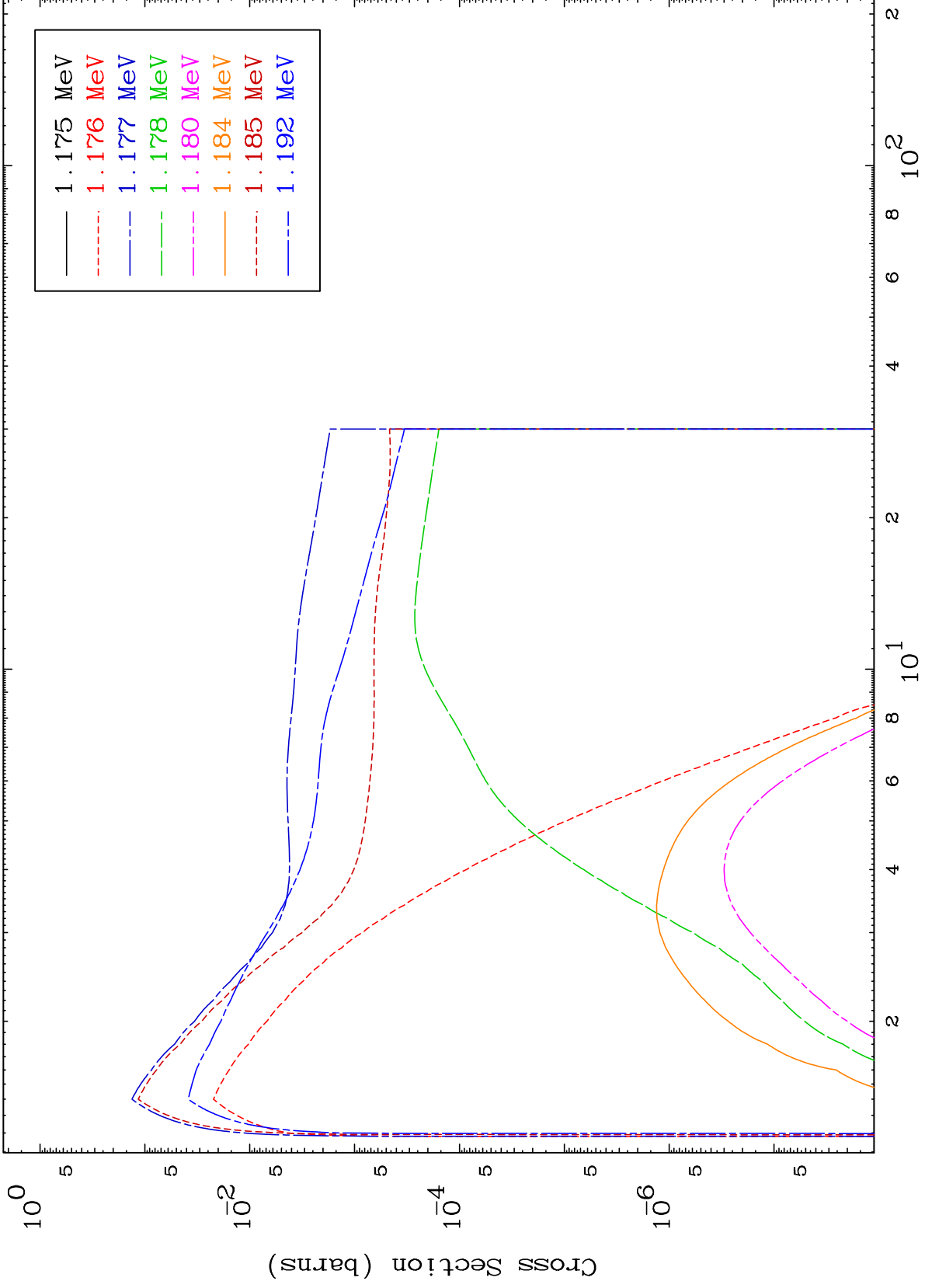
Incident Energy (MeV)

104-Rf-257

MAT 457

(n,n') Level
293 Kelvin Cross Sections

104-Rf-257



11

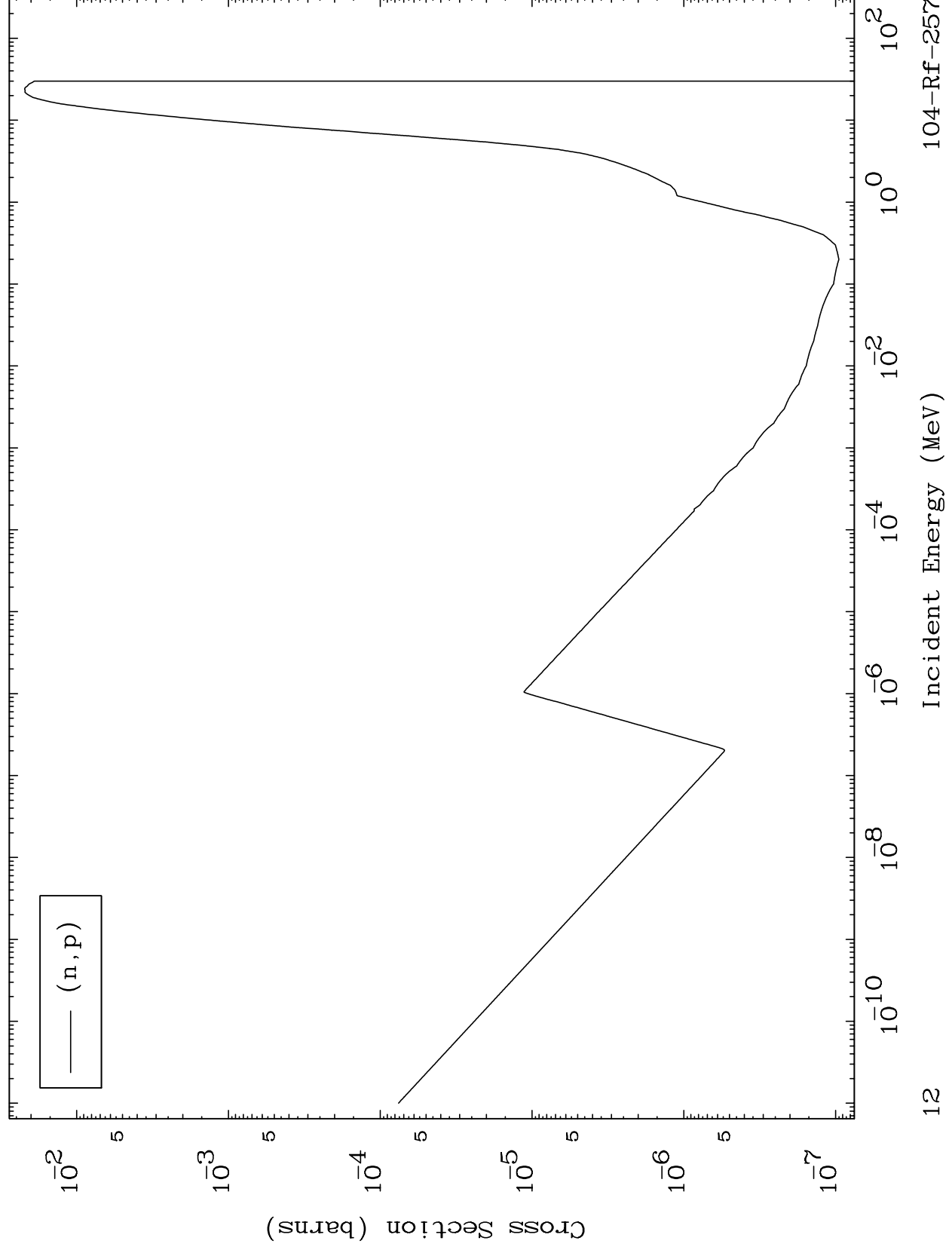
Incident Energy (MeV)

104-Rf-257

MAT 457

(n,p) Levels
293 Kelvin Cross Sections

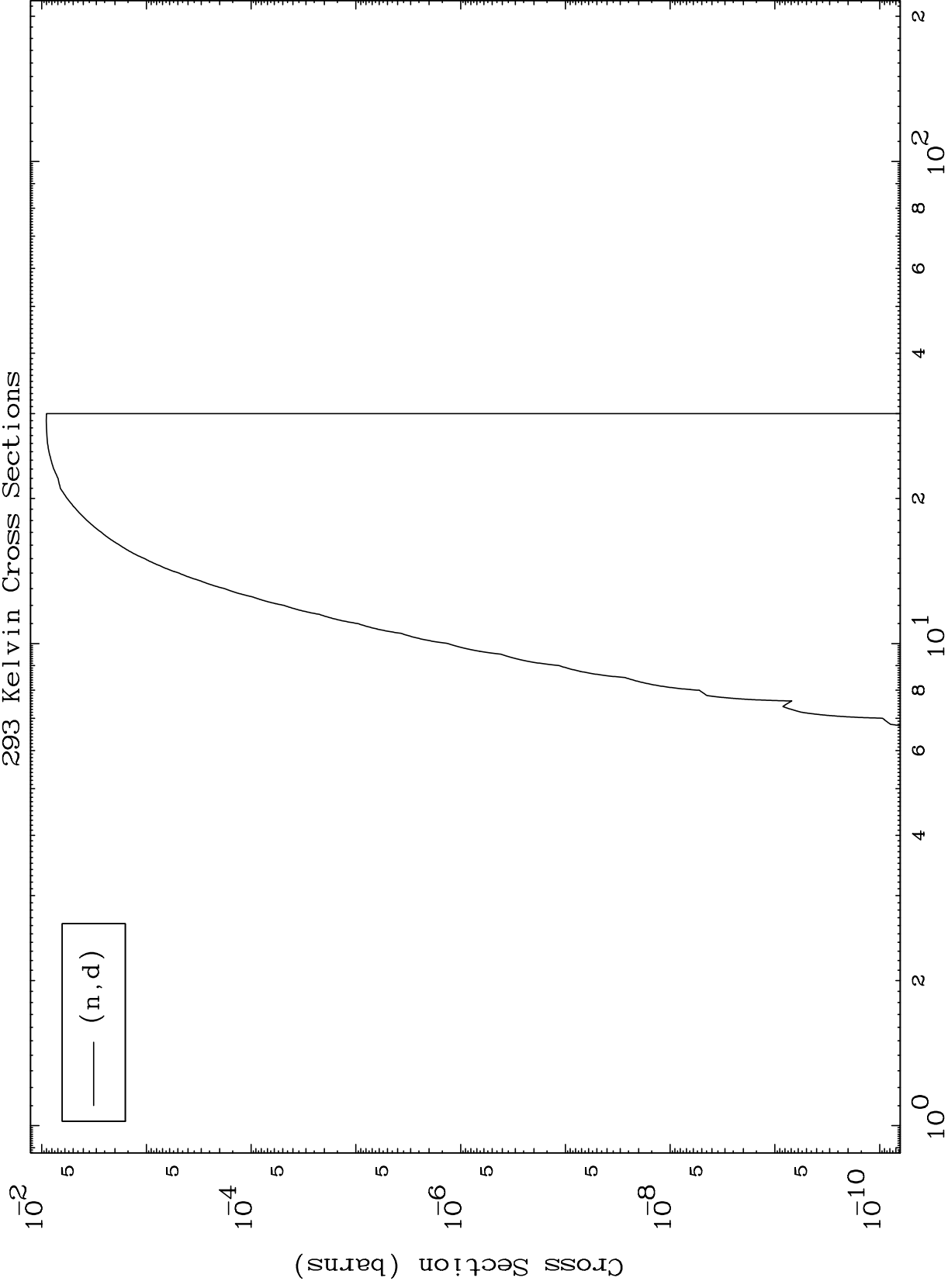
104-Rf-257



MAT 457

(n,d) Levels
293 Kelvin Cross Sections

104-Rf-257



13

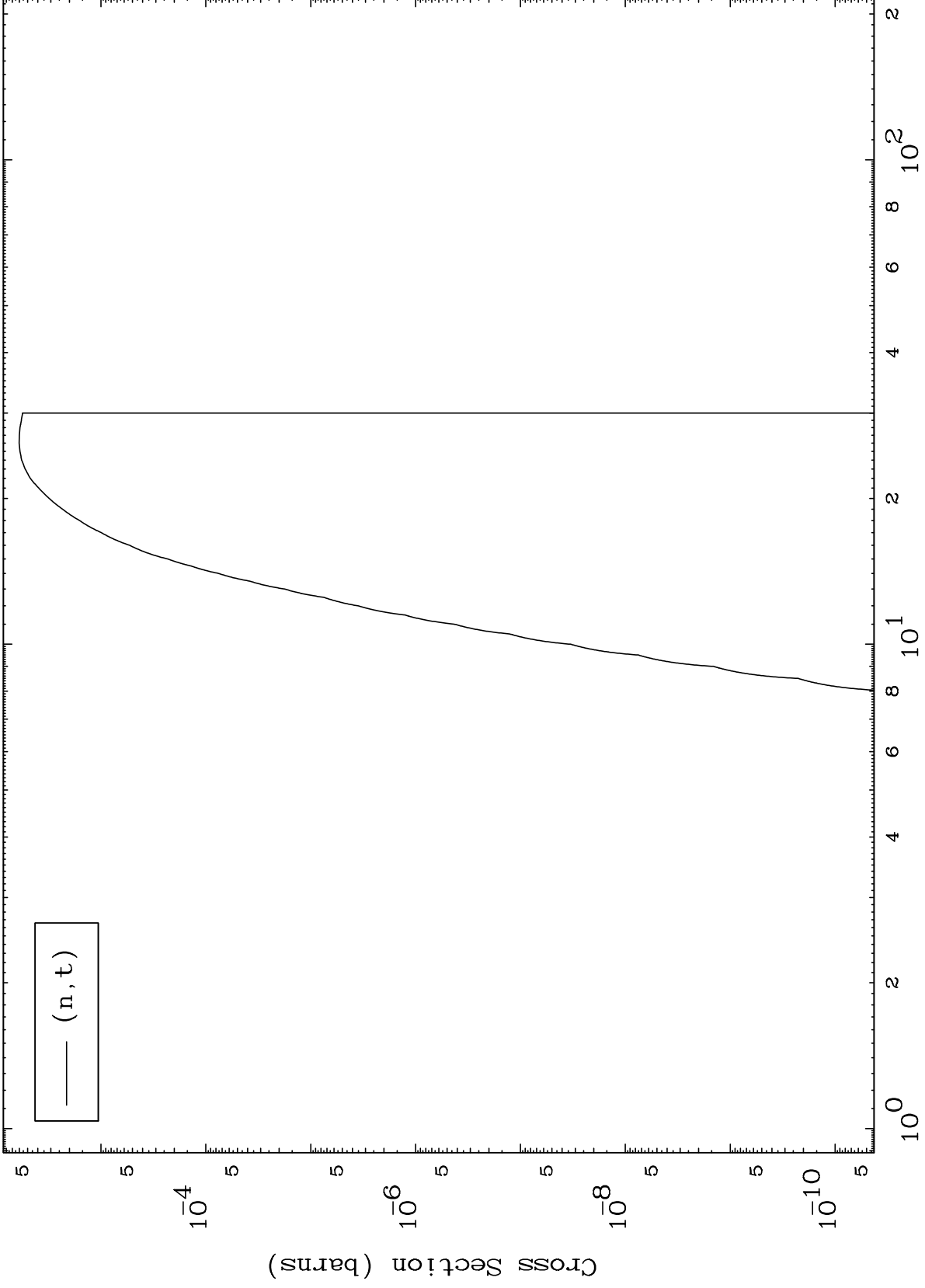
Incident Energy (MeV)

104-Rf-257

MAT 457

(n,t) Levels
293 Kelvin Cross Sections

104-Rf-257



14

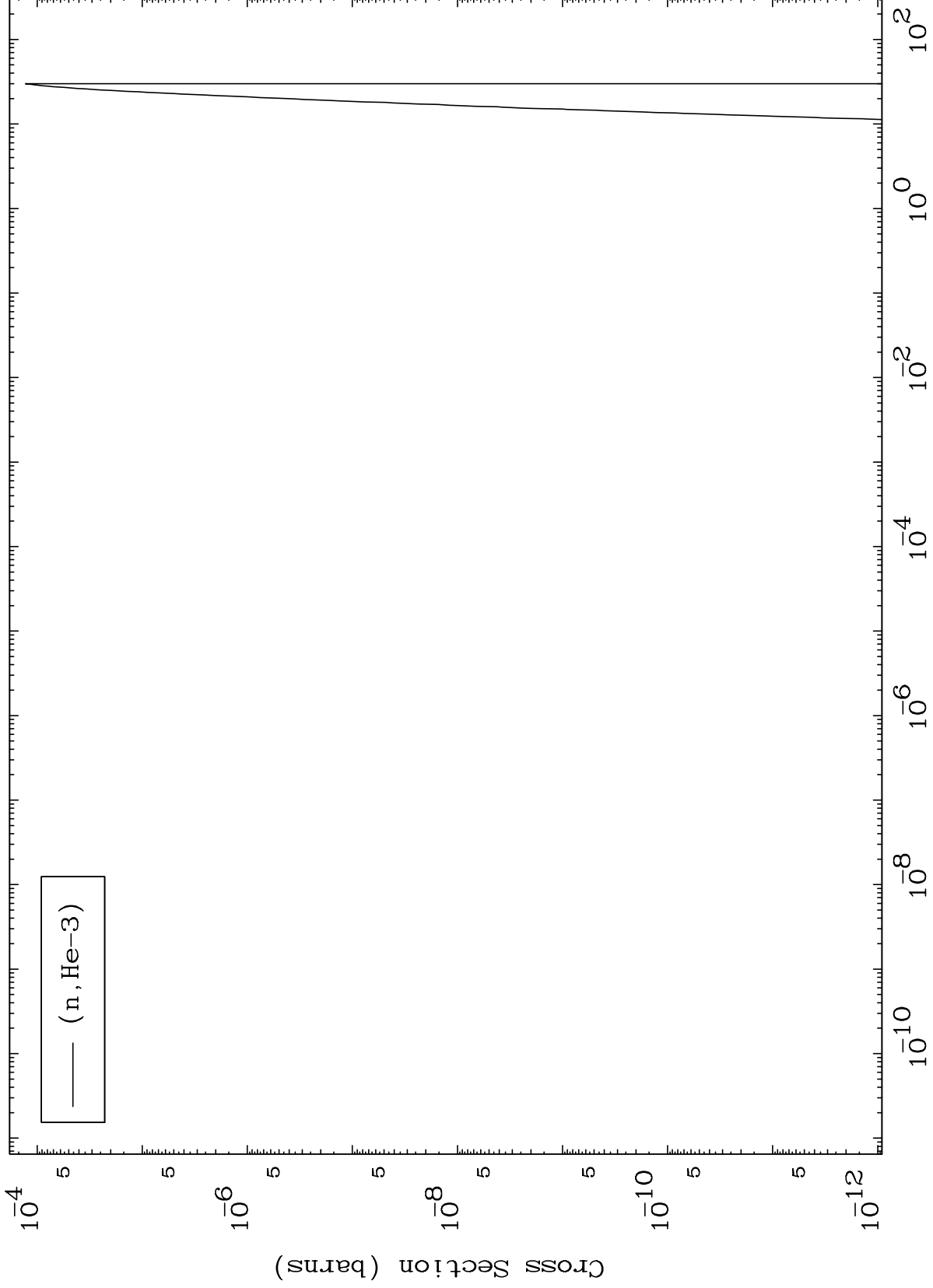
Incident Energy (MeV)

104-Rf-257

MAT 457

(n,He3) Levels
293 Kelvin Cross Sections

104-Rf-257



15

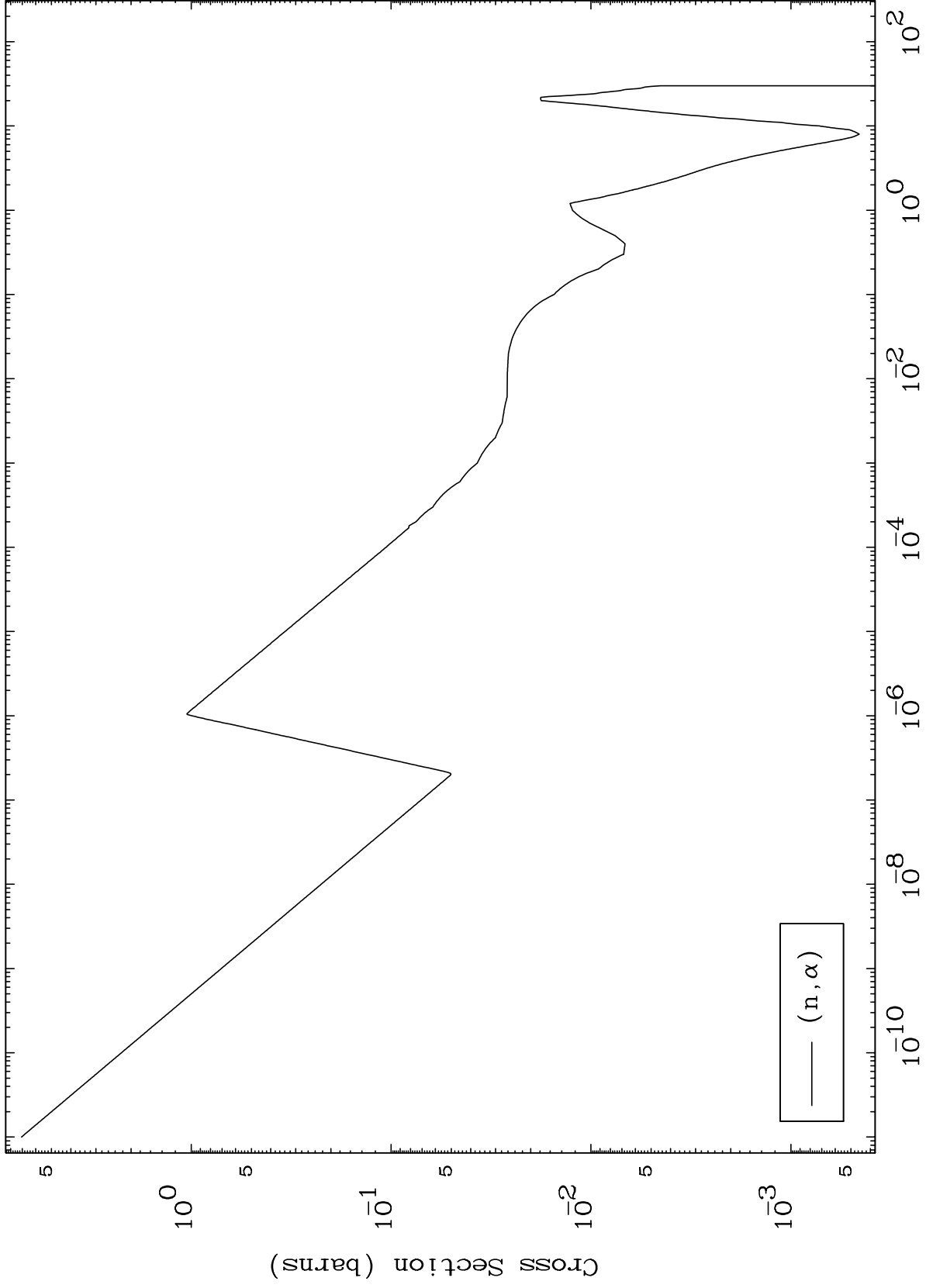
Incident Energy (MeV)

104-Rf-257

MAT 457

(n,α) Levels
293 Kelvin Cross Sections

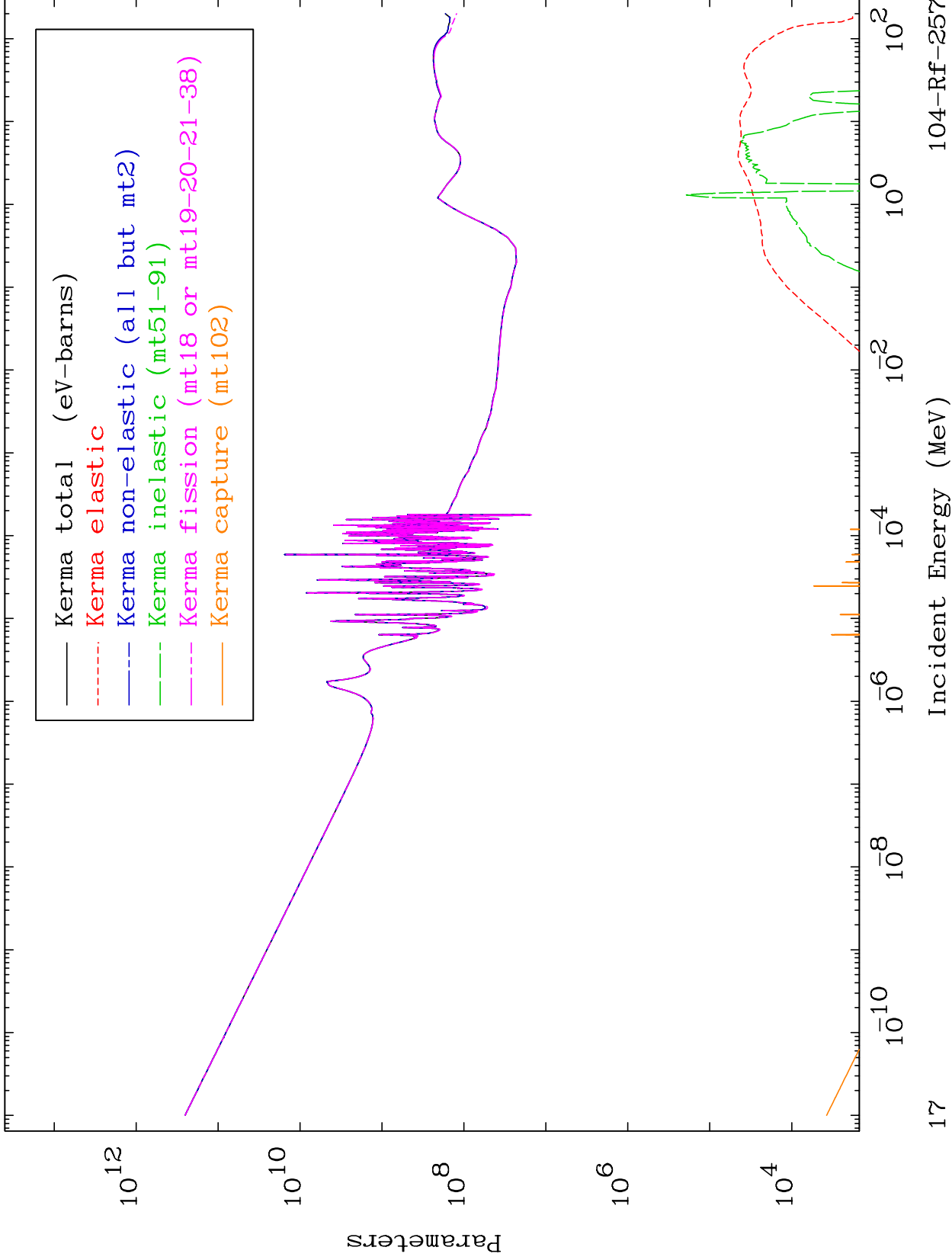
104-Rf-257

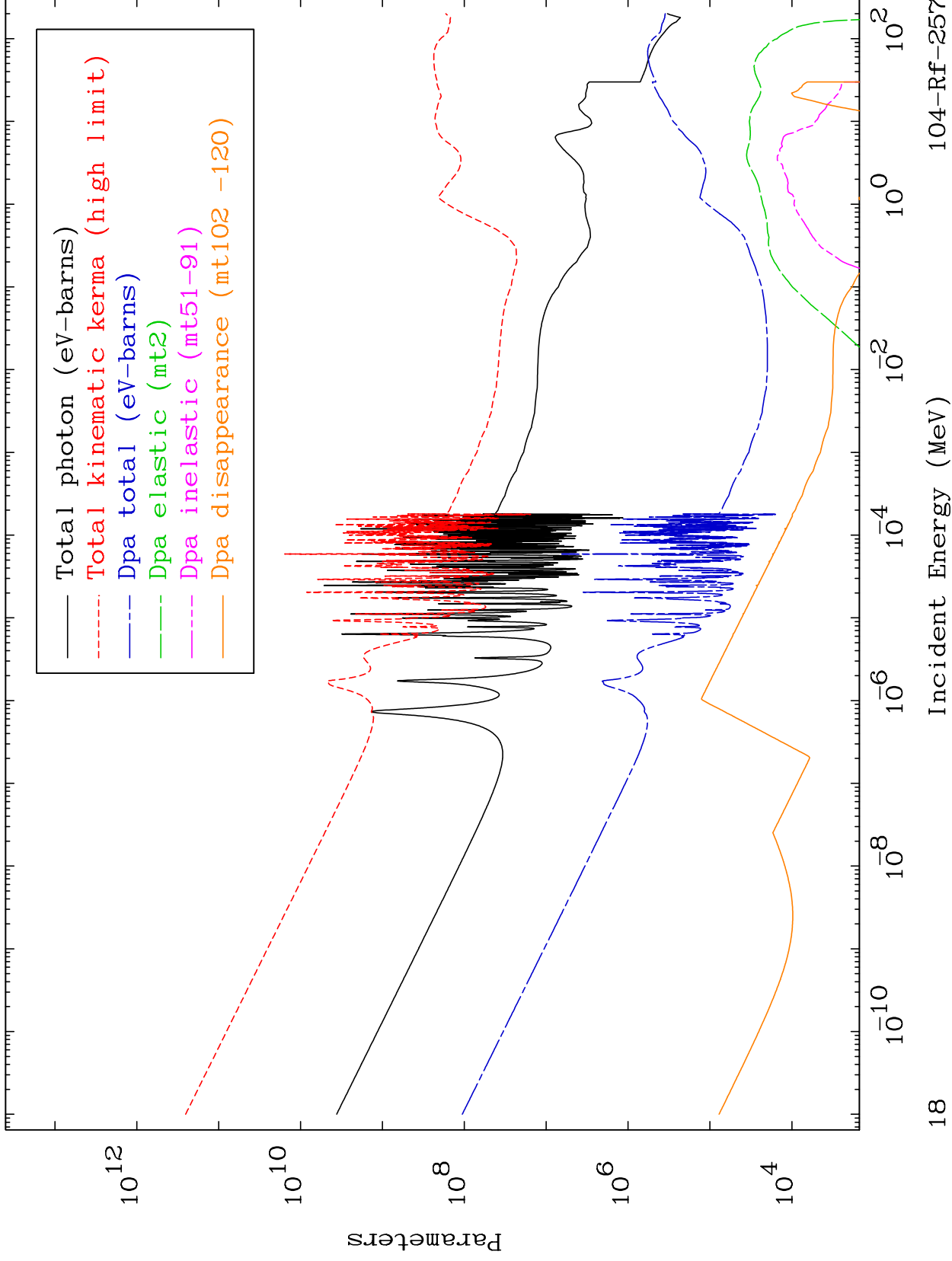


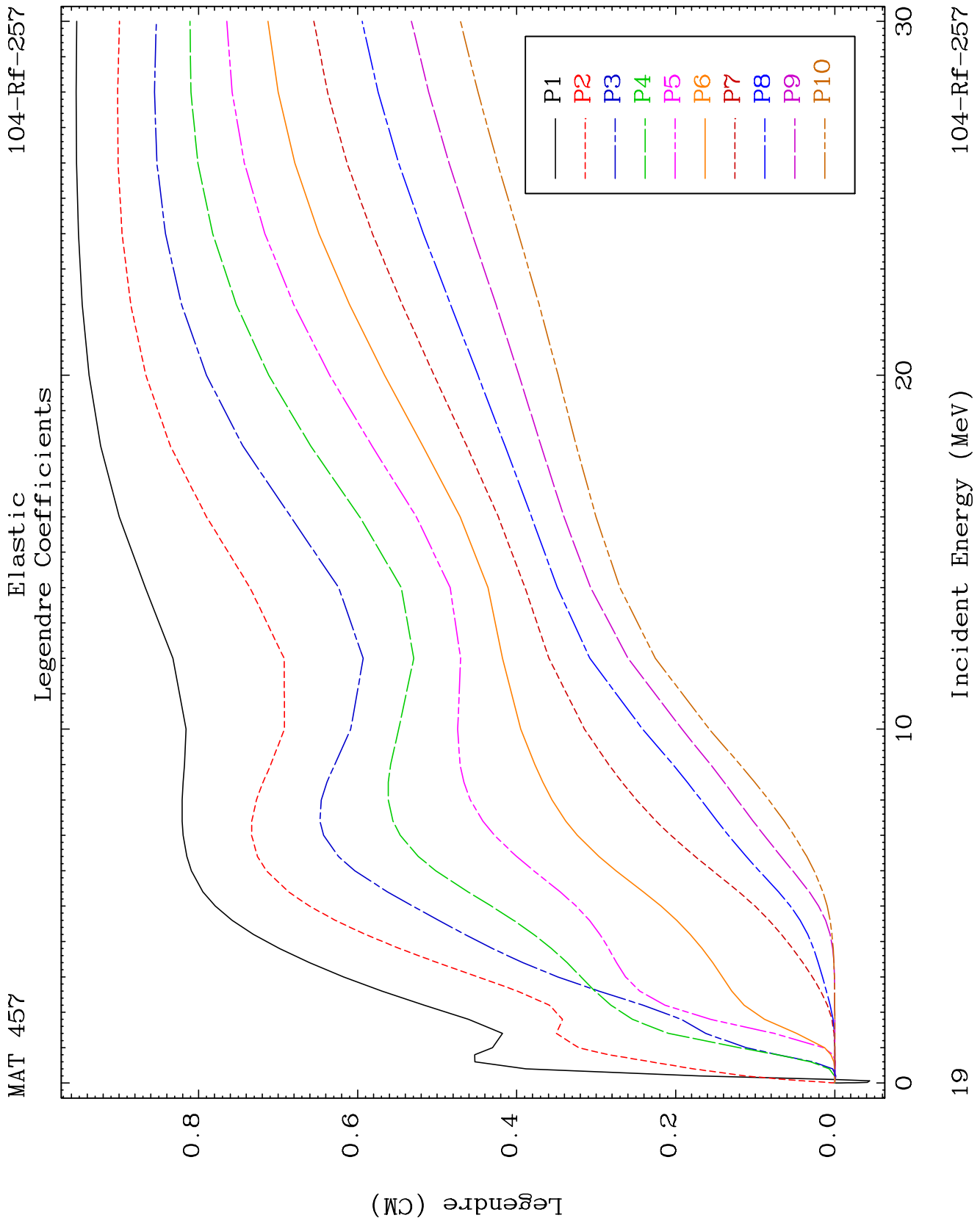
16

Incident Energy (MeV)

104-Rf-257



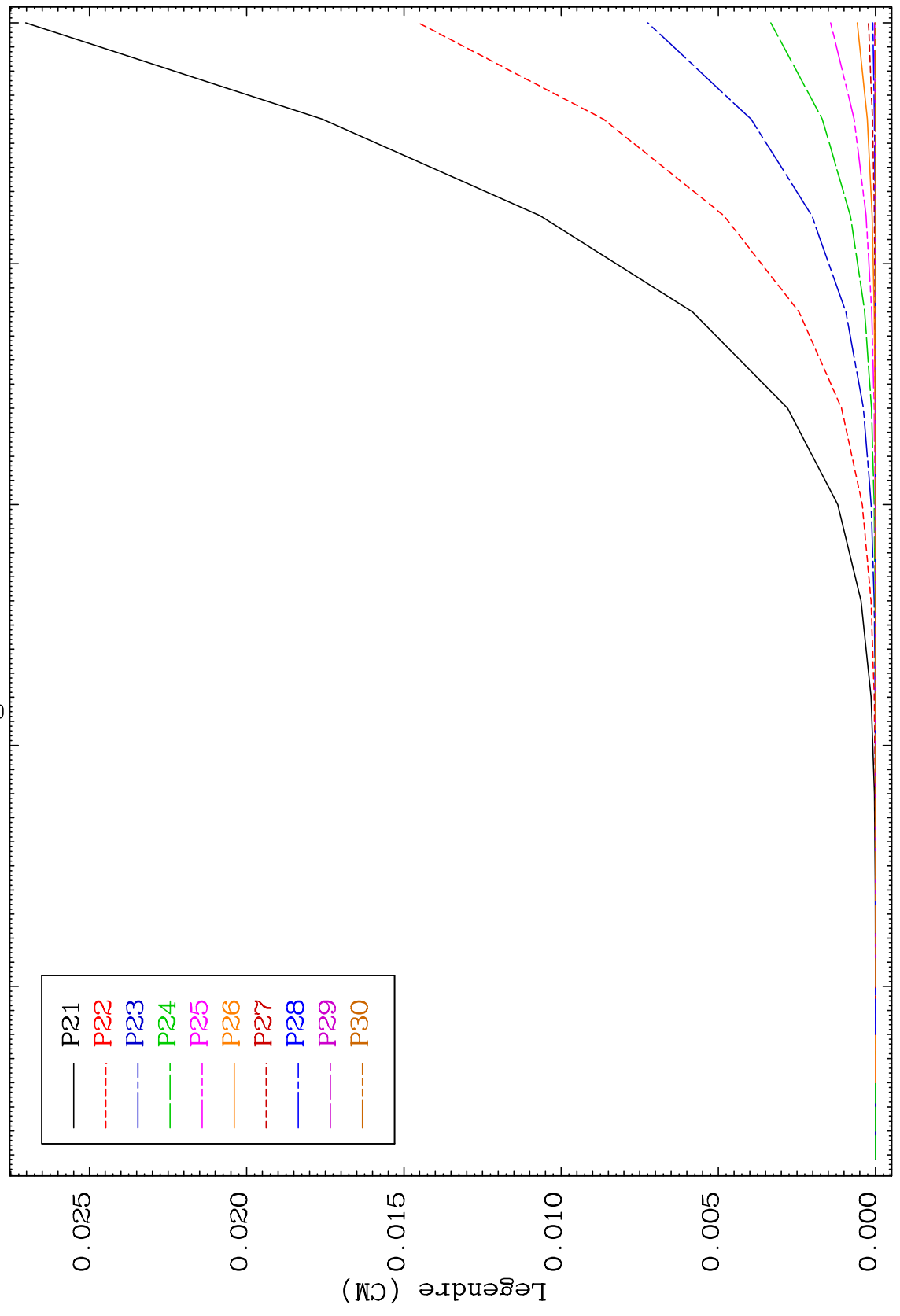




MAT 457

Elastic Legendre Coefficients

104-Rf-257



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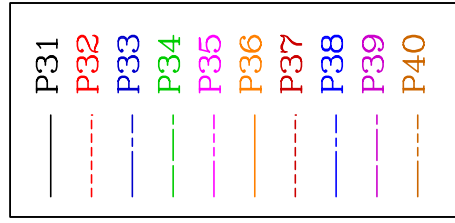
Incident Energy (MeV)

104-Rf-257

MAT 457

Elastic Legendre Coefficients

104-Rf-257



$\times 10^{-6}$

Legendre (CM)

3

2

1

0

15

20

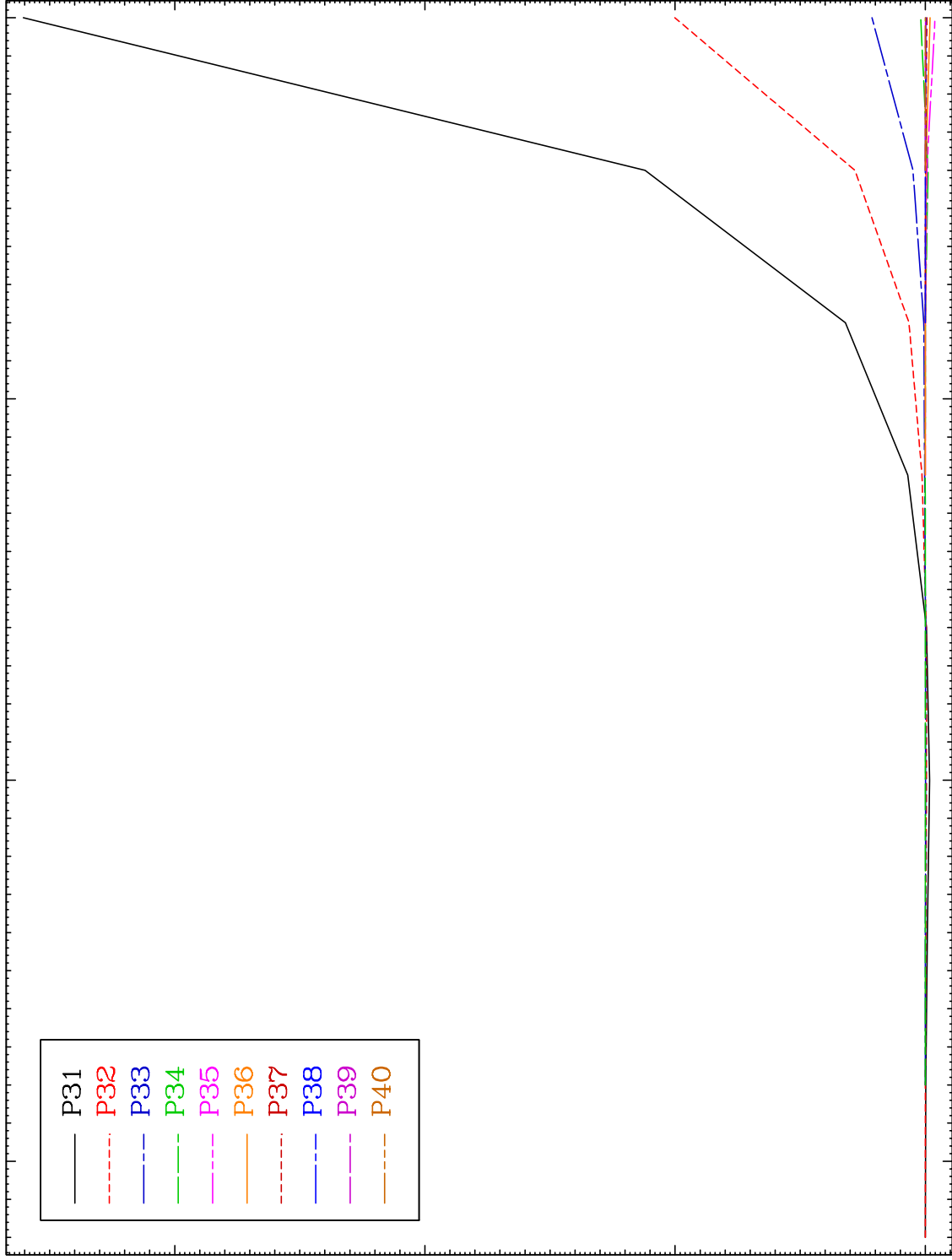
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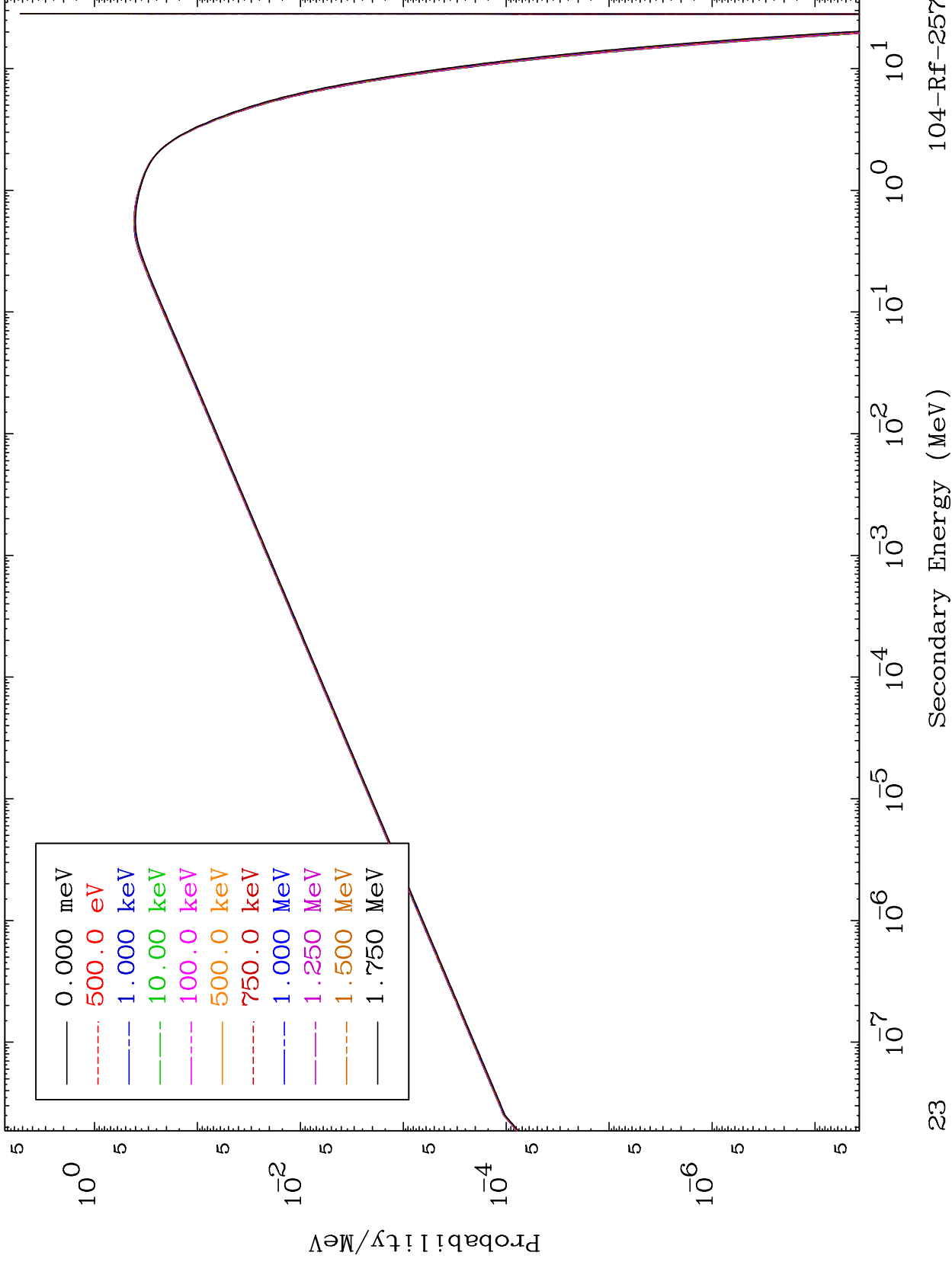
30

22

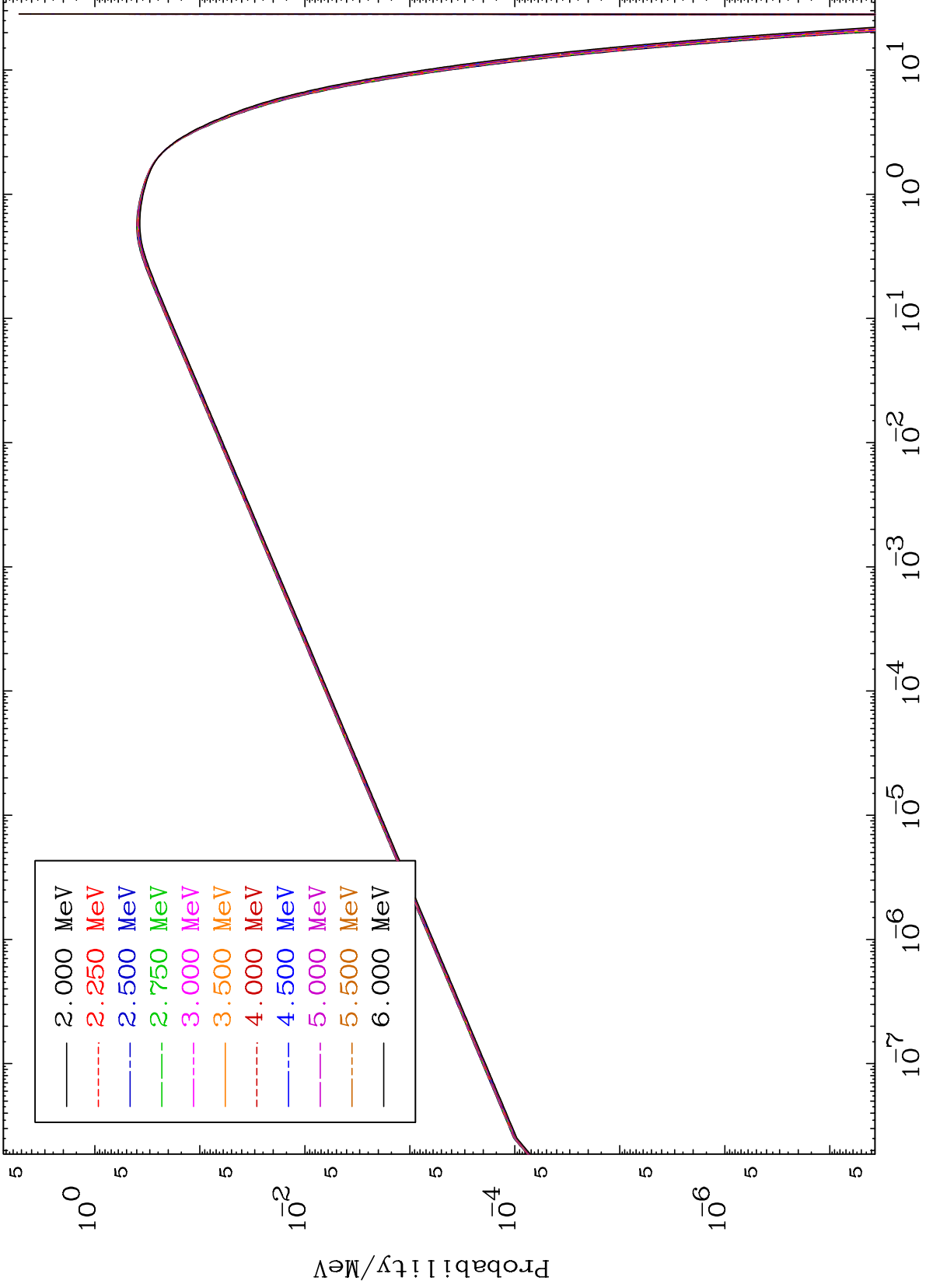
Incident Energy (MeV)

104-Rf-257

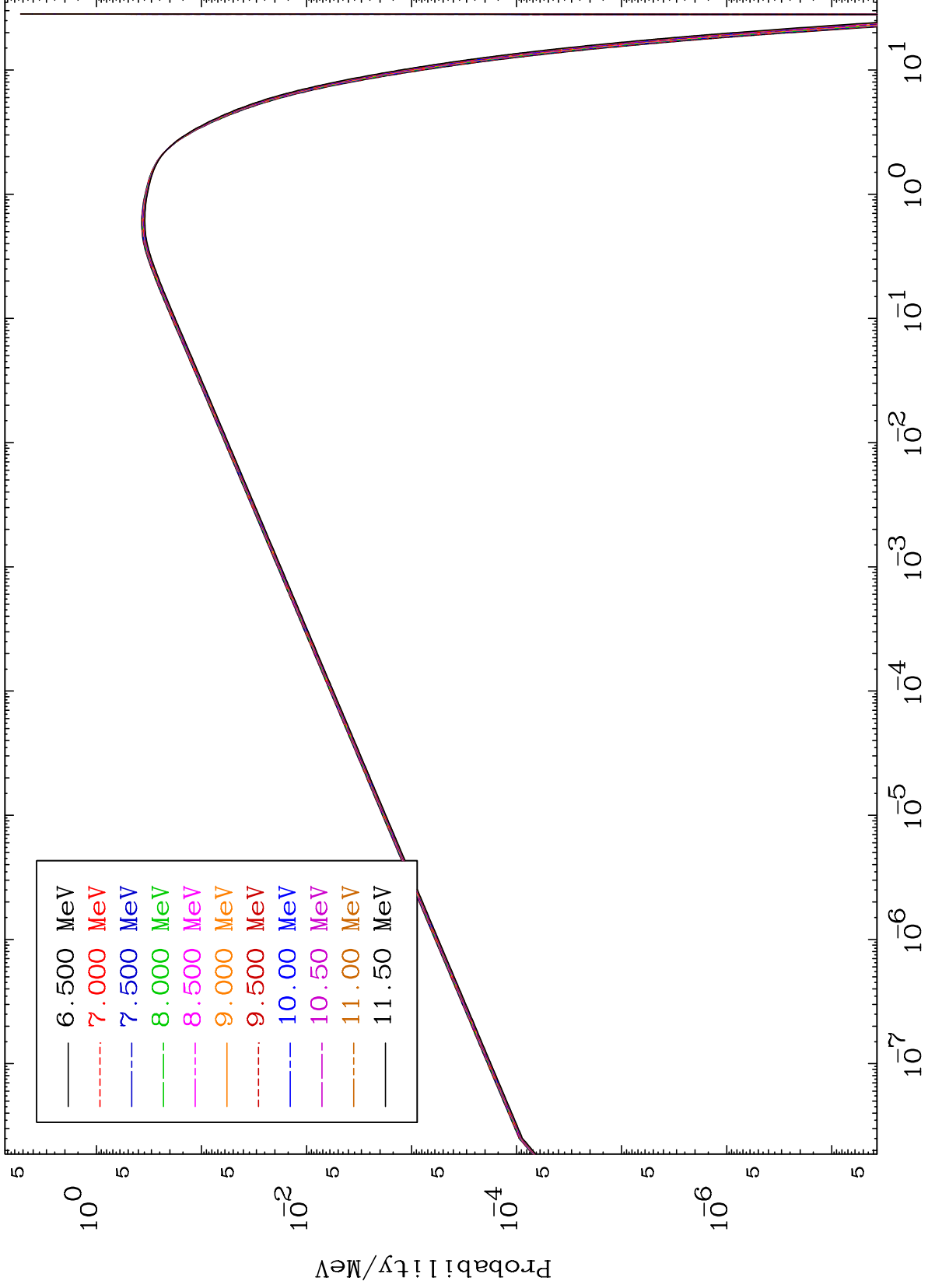


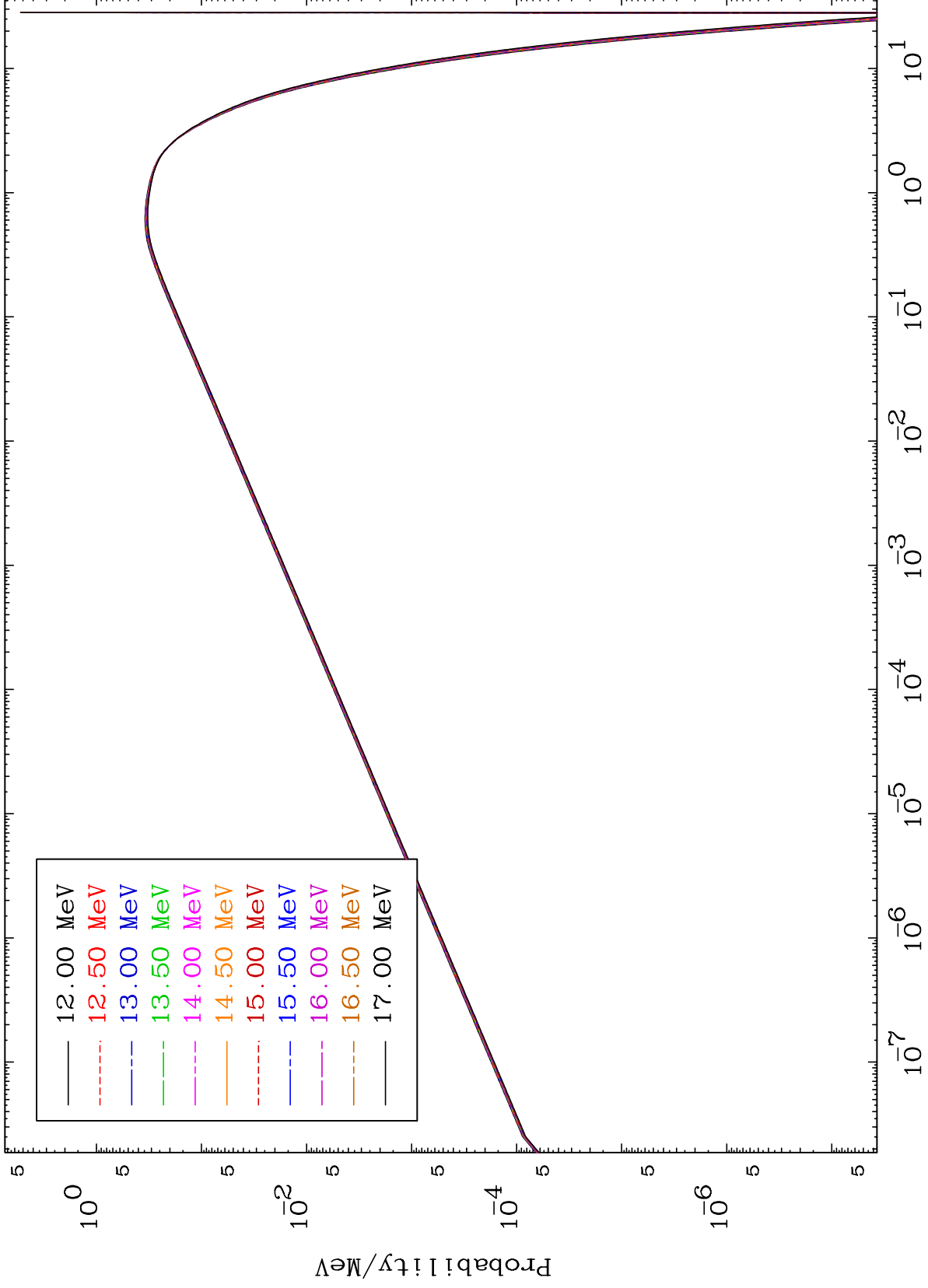


Fission
Energy Distribution

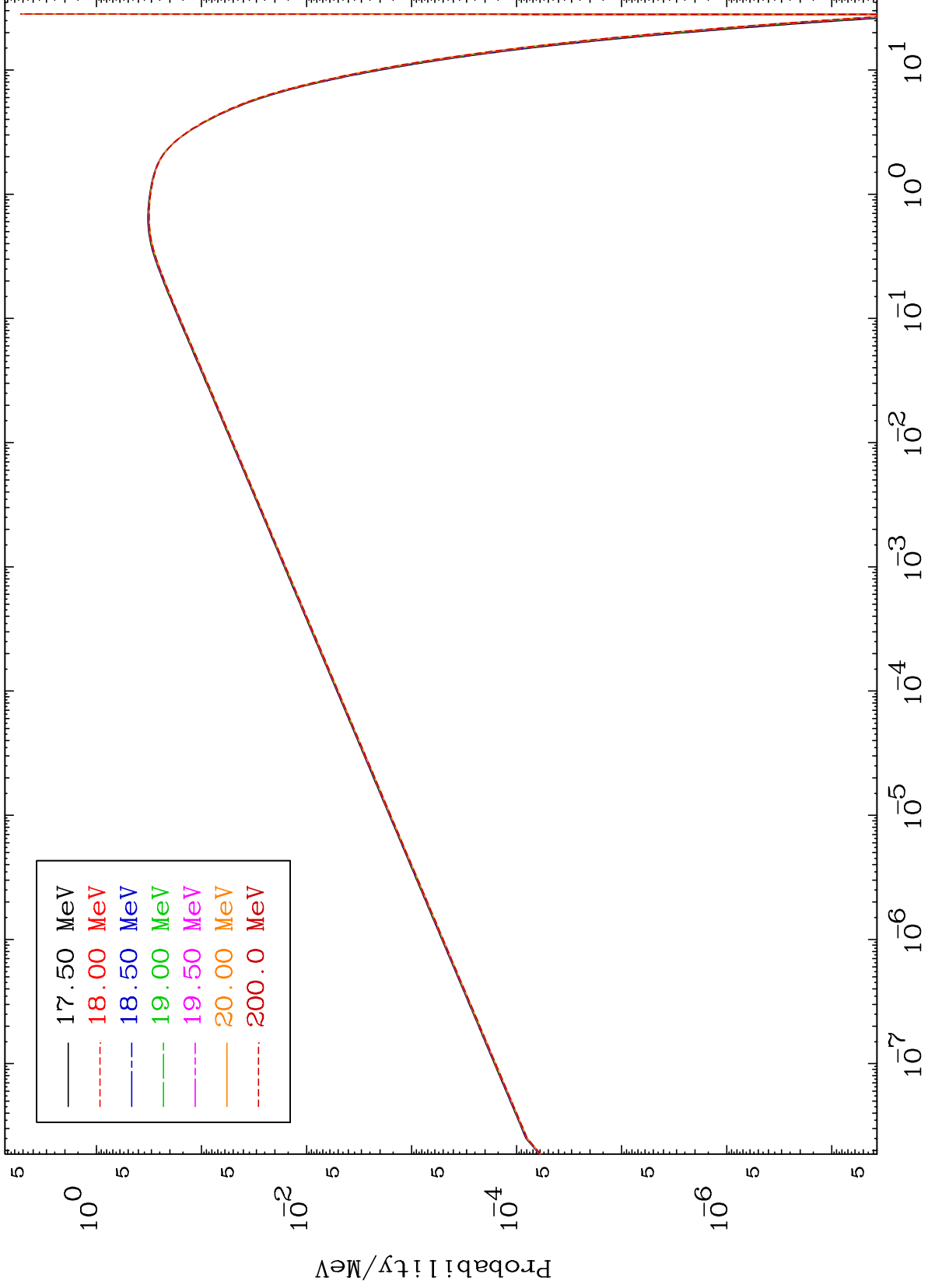


Fission
Energy Distribution





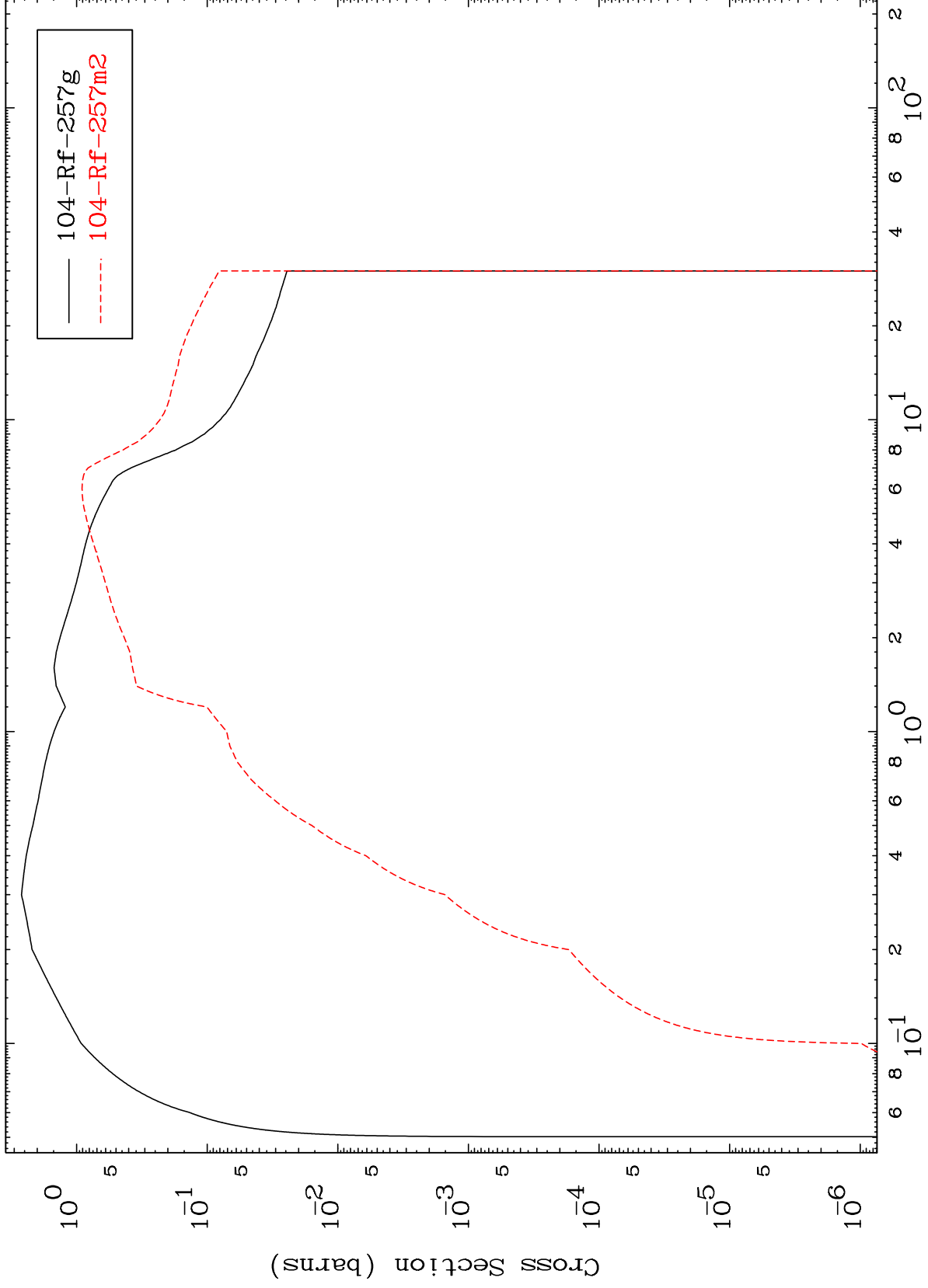
Fission
Energy Distribution



MAT 457

104-Rf-257

Inelastic
Radionuclide Production Cross Section



28

Incident Energy (MeV)

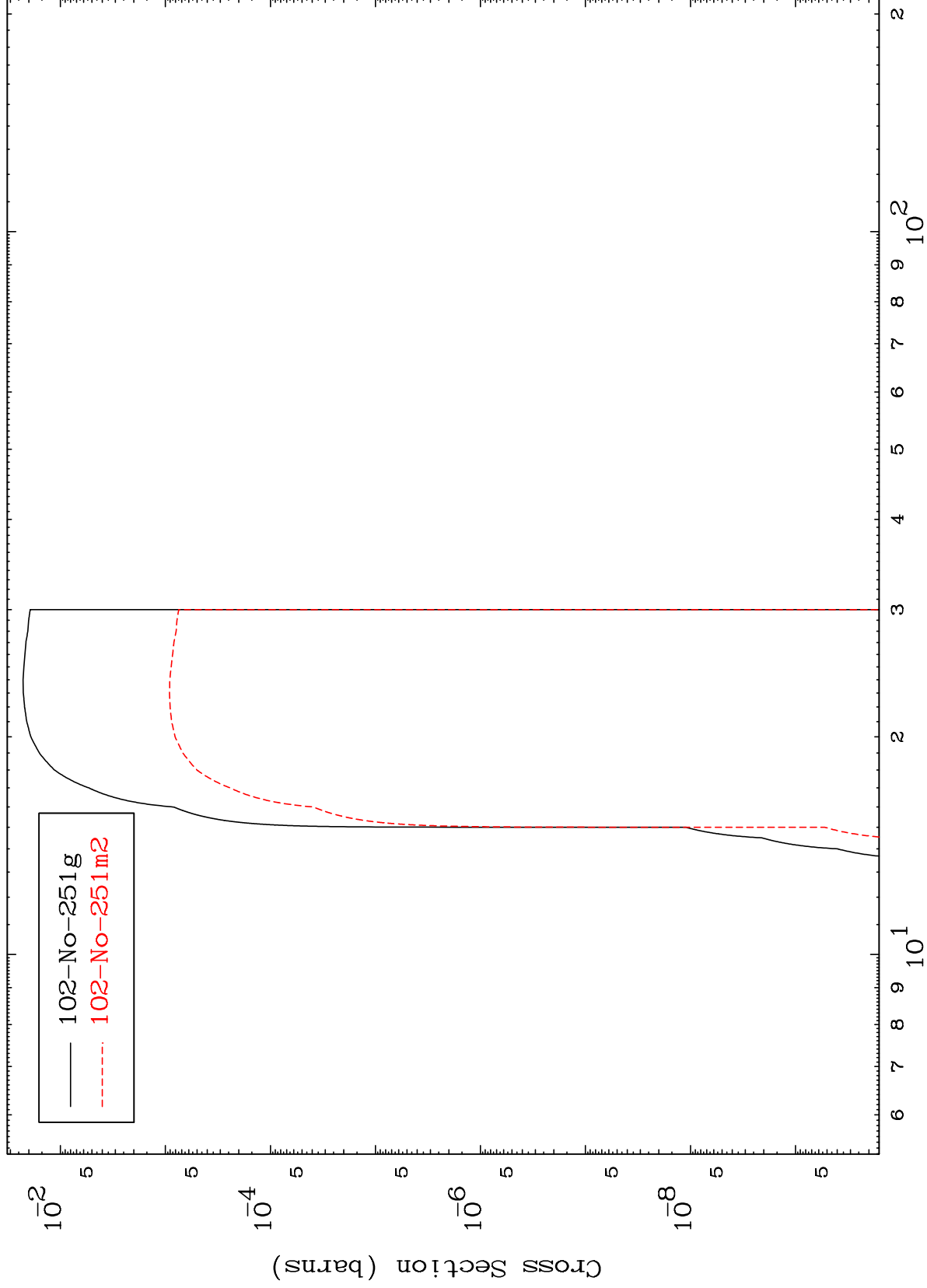
104-Rf-257

MAT 457

(n,3n) α

104-Rf-257

Radionuclide Production Cross Section



29

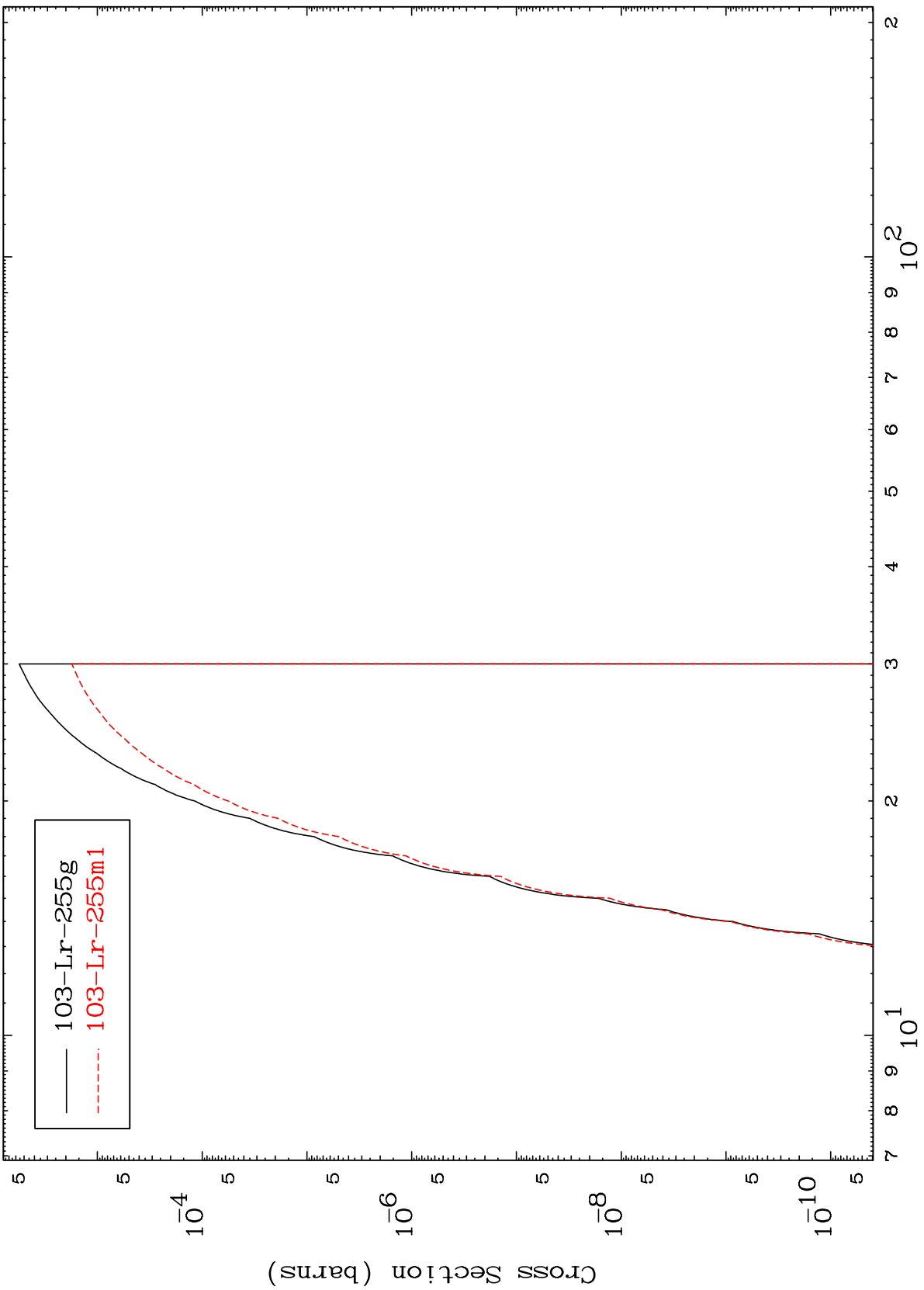
Incident Energy (MeV)

104-Rf-257

MAT 457

104-Rf-257

(n,n') d
Radionuclide Production Cross Section



30

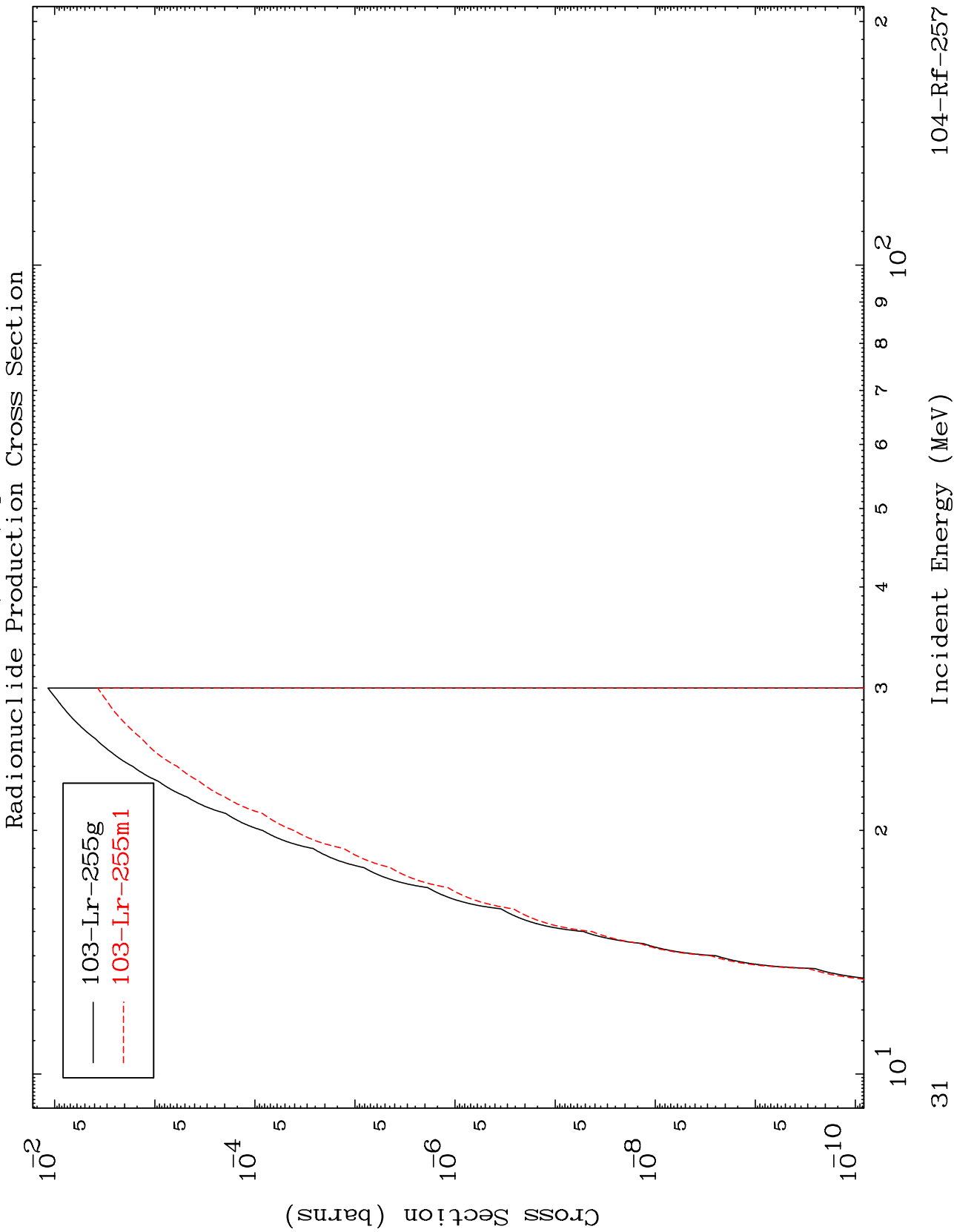
Incident Energy (MeV)

104-Rf-257

MAT 457

(n,2n) p

104-Rf-257

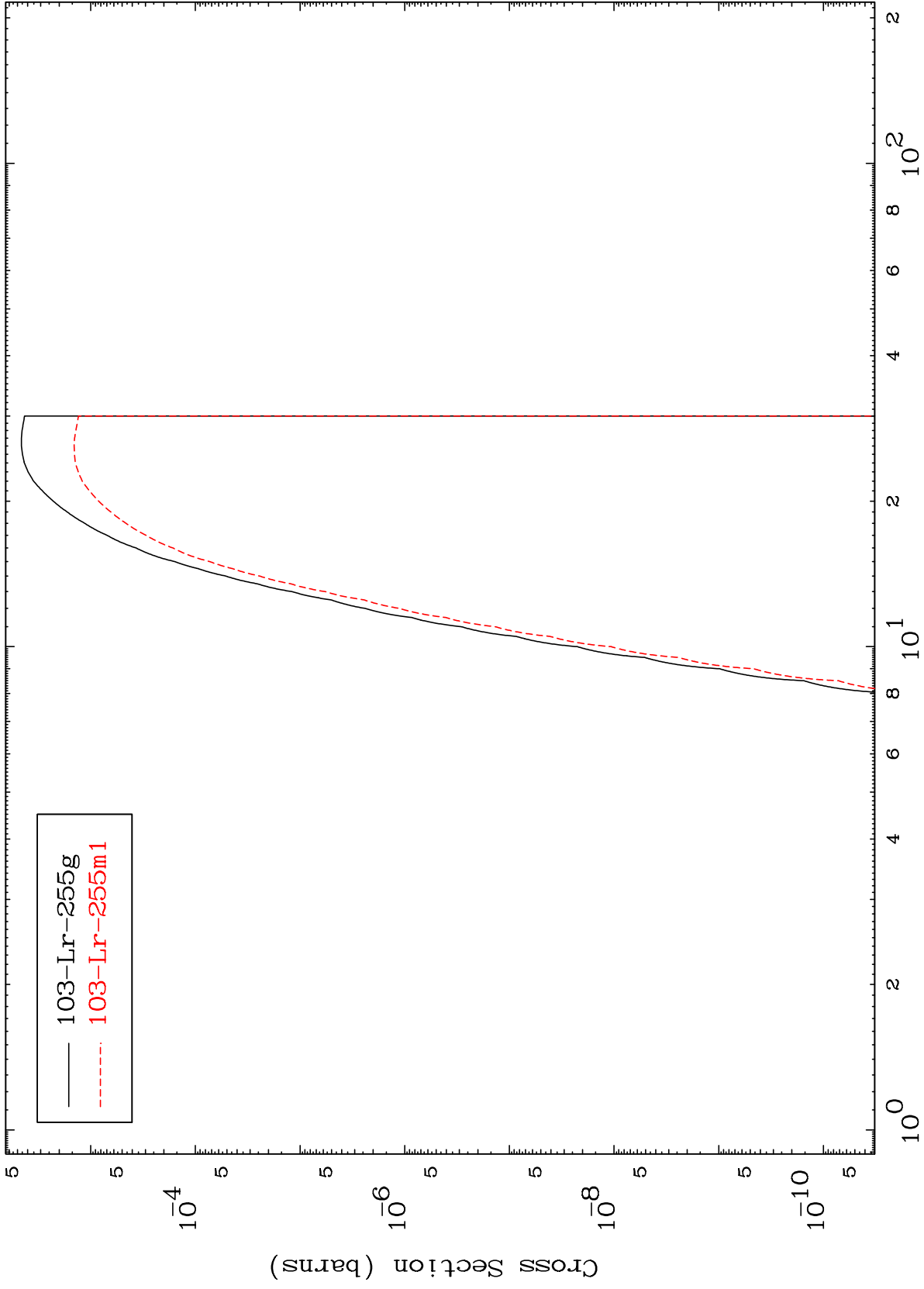


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MAT 457

104-Rf-257

(n, t)
Radionuclide Production Cross Section



104-Rf-257

Incident Energy (MeV)

32

MAT 457

104-Rf-257

Radionuclide Production Cross Section
(n,2α)

