

Program EVALPLOT
(Version 2015-2)

by

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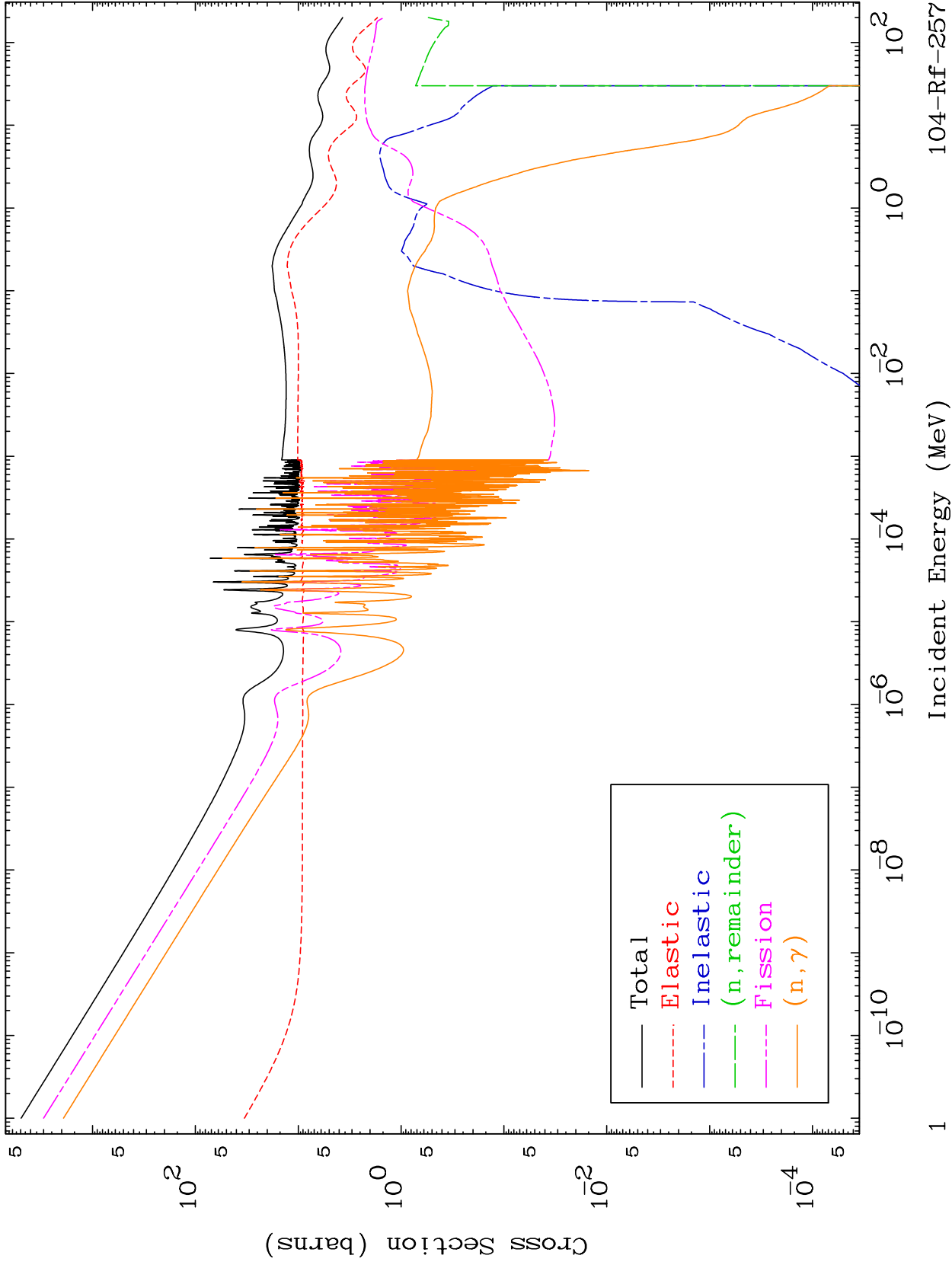
Press Mouse Button to Start

MAT 457

Major

293 Kelvin Cross Sections

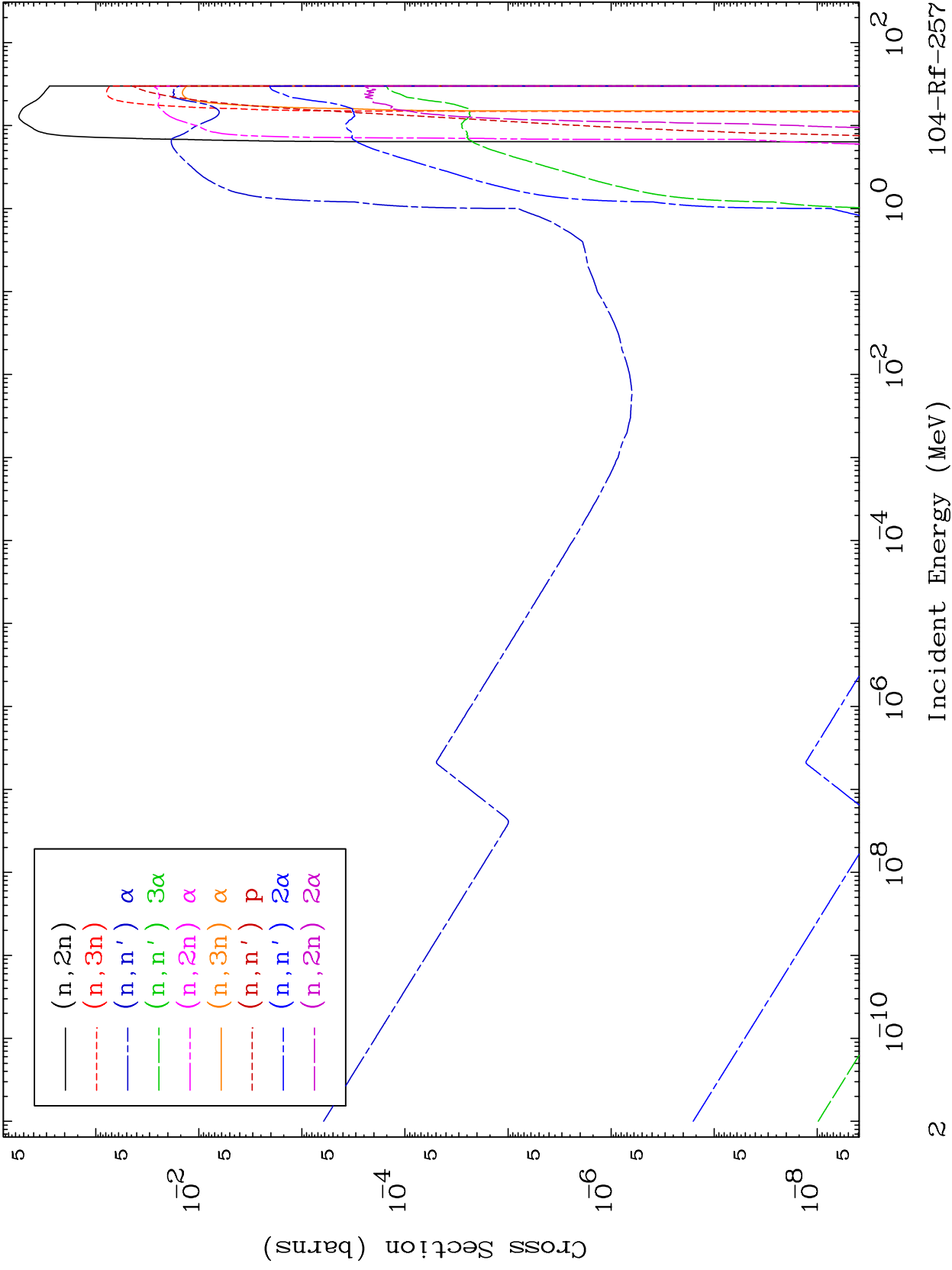
104-Rf-257



MAT 457

Neutron Production
293 Kelvin Cross Sections

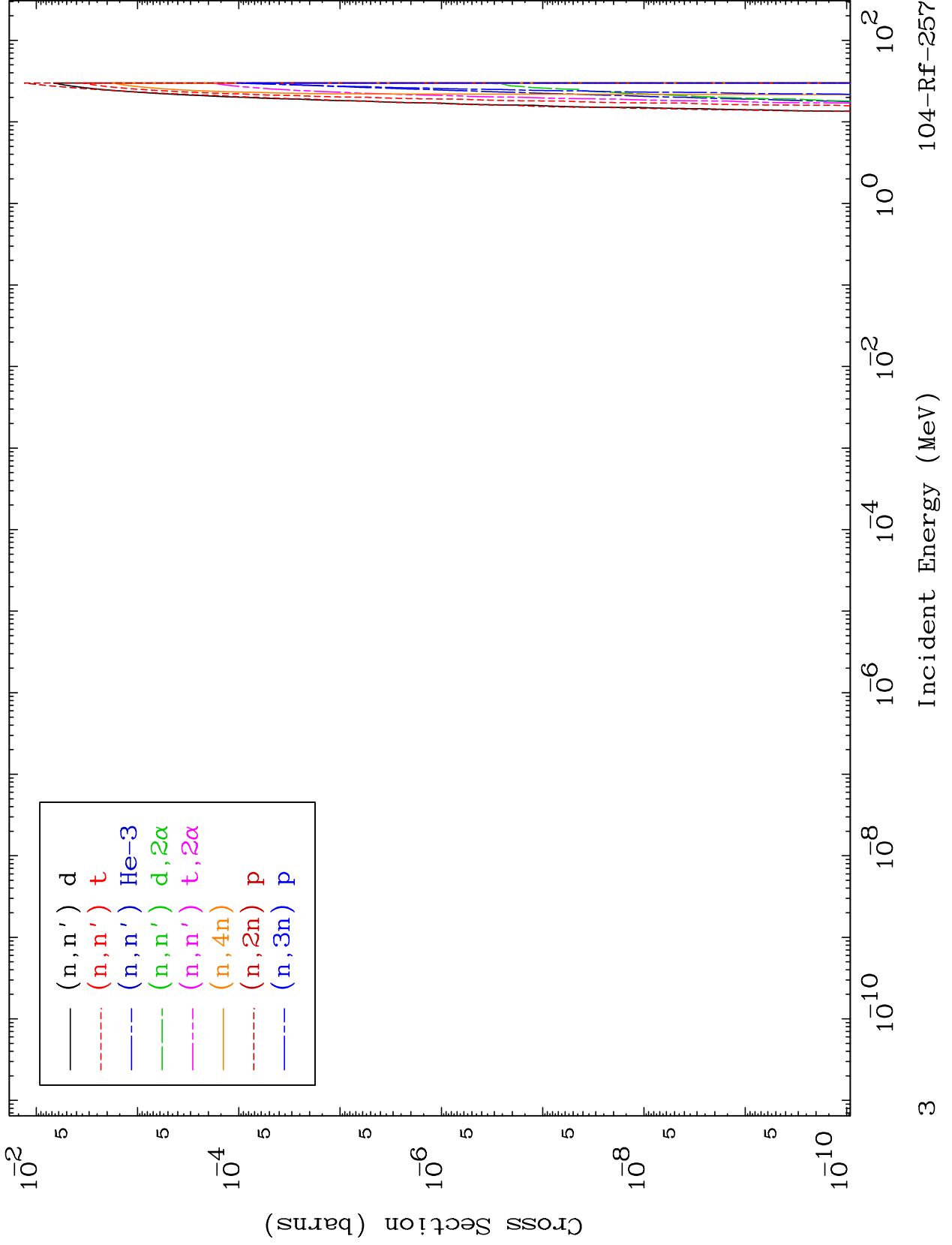
104-Rf-257



MAT 457

Neutron Production
293 Kelvin Cross Sections

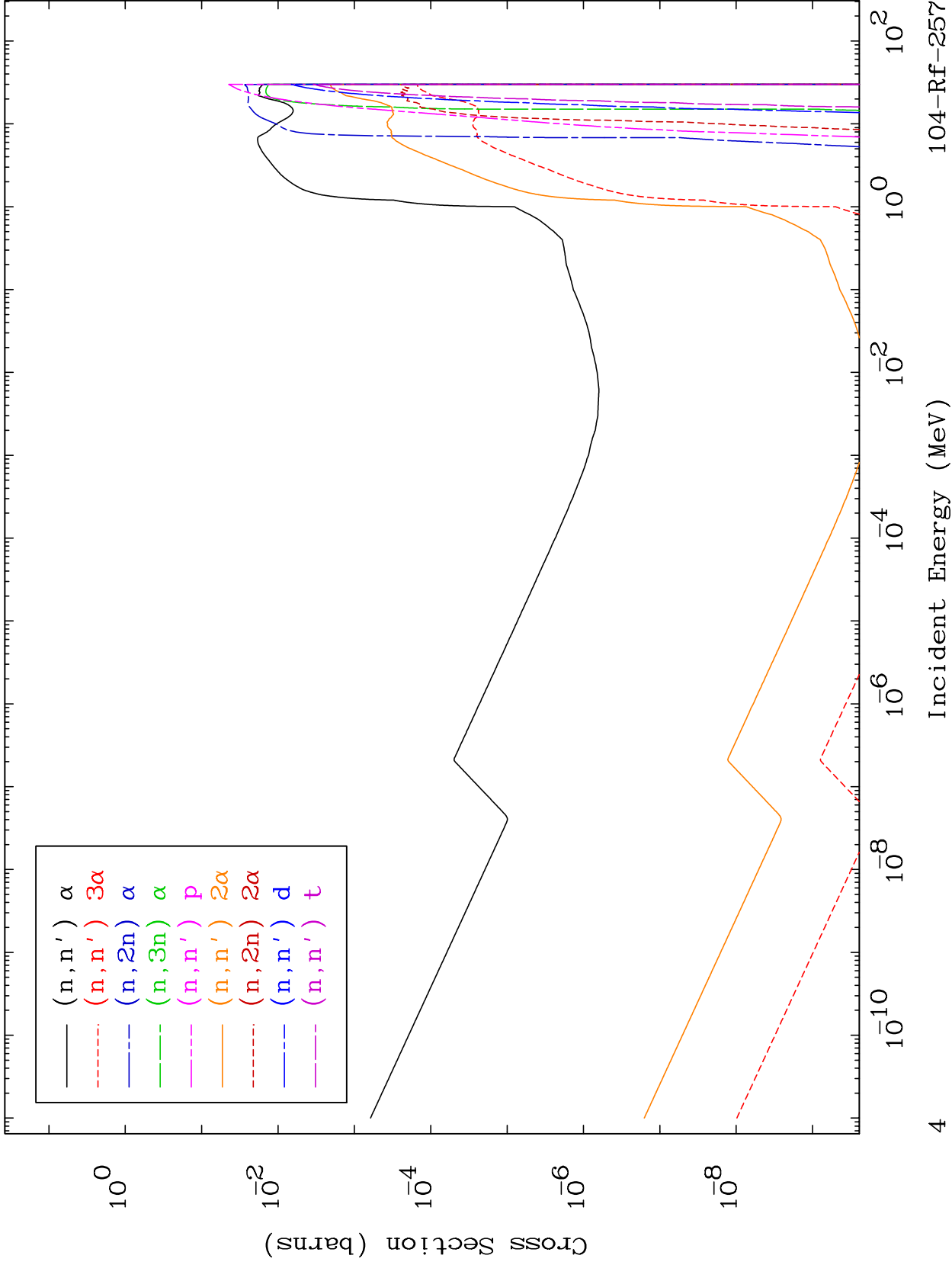
104-Rf-257



MAT 457

Charged Particle
293 Kelvin Cross Sections

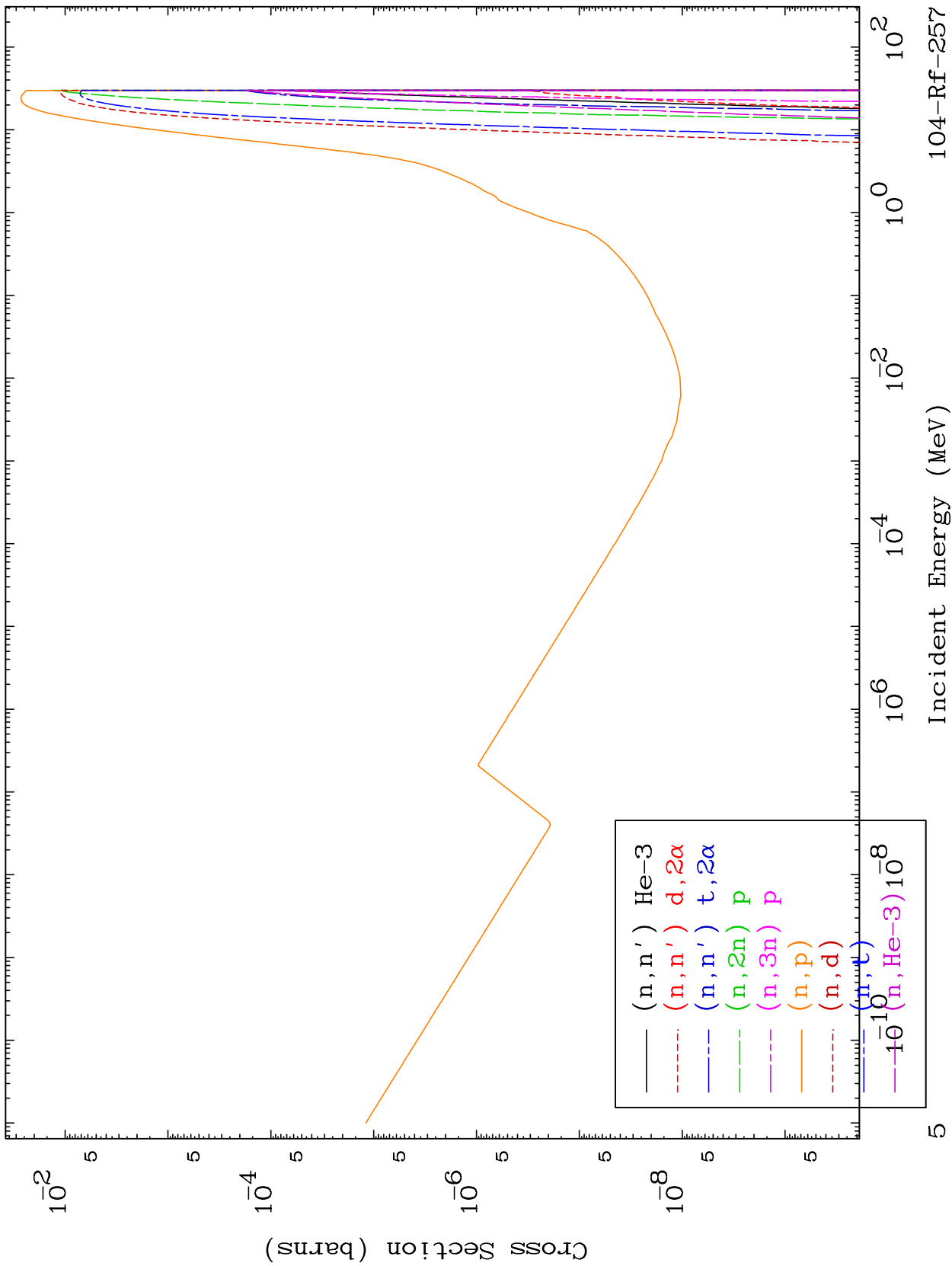
104-Rf-257



MAT 457

Charged Particle
293 Kelvin Cross Sections

104-Rf-257



104-Rf-257

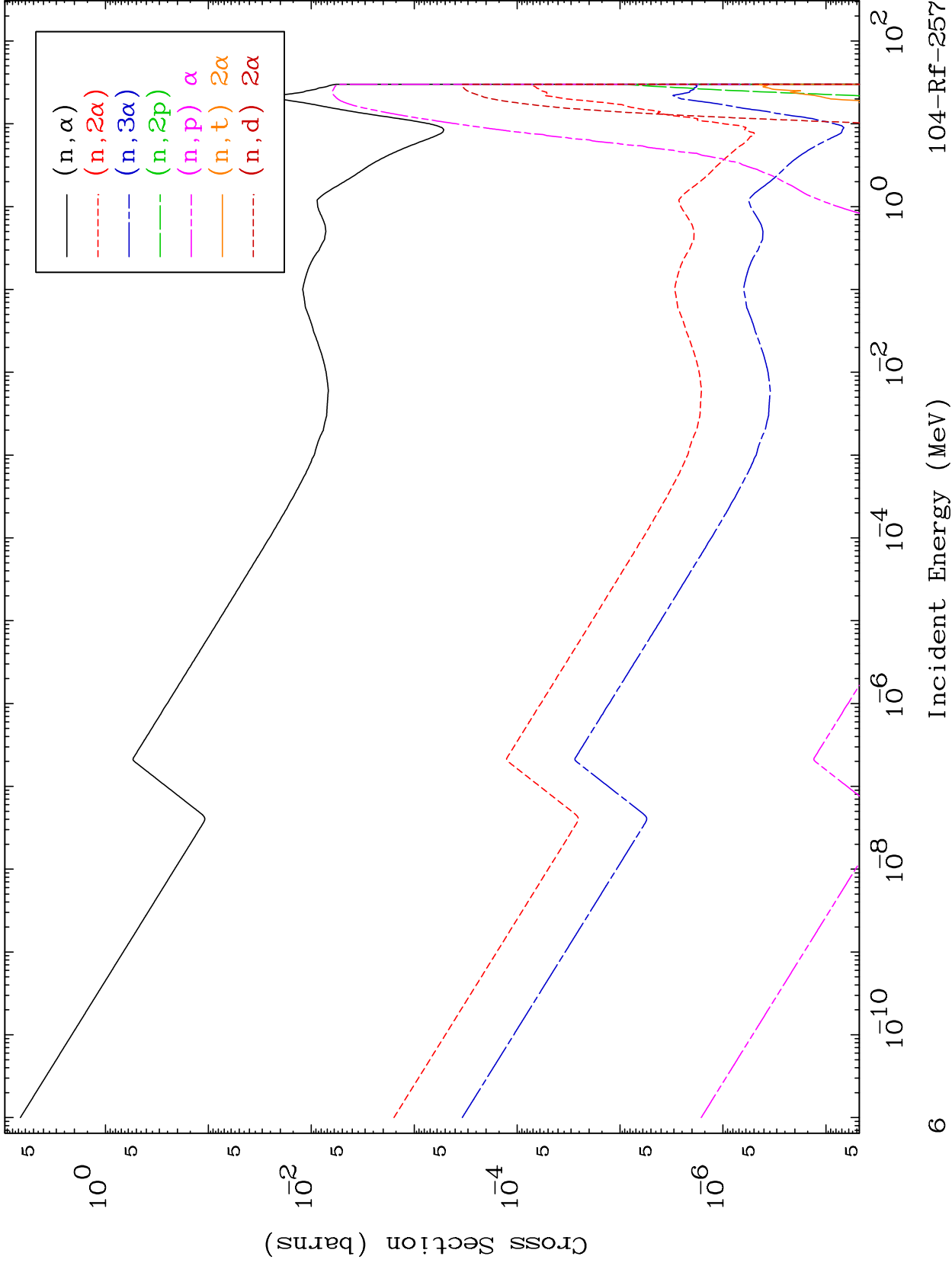
Incident Energy (MeV)

5

MAT 457

Charged Particle
293 Kelvin Cross Sections

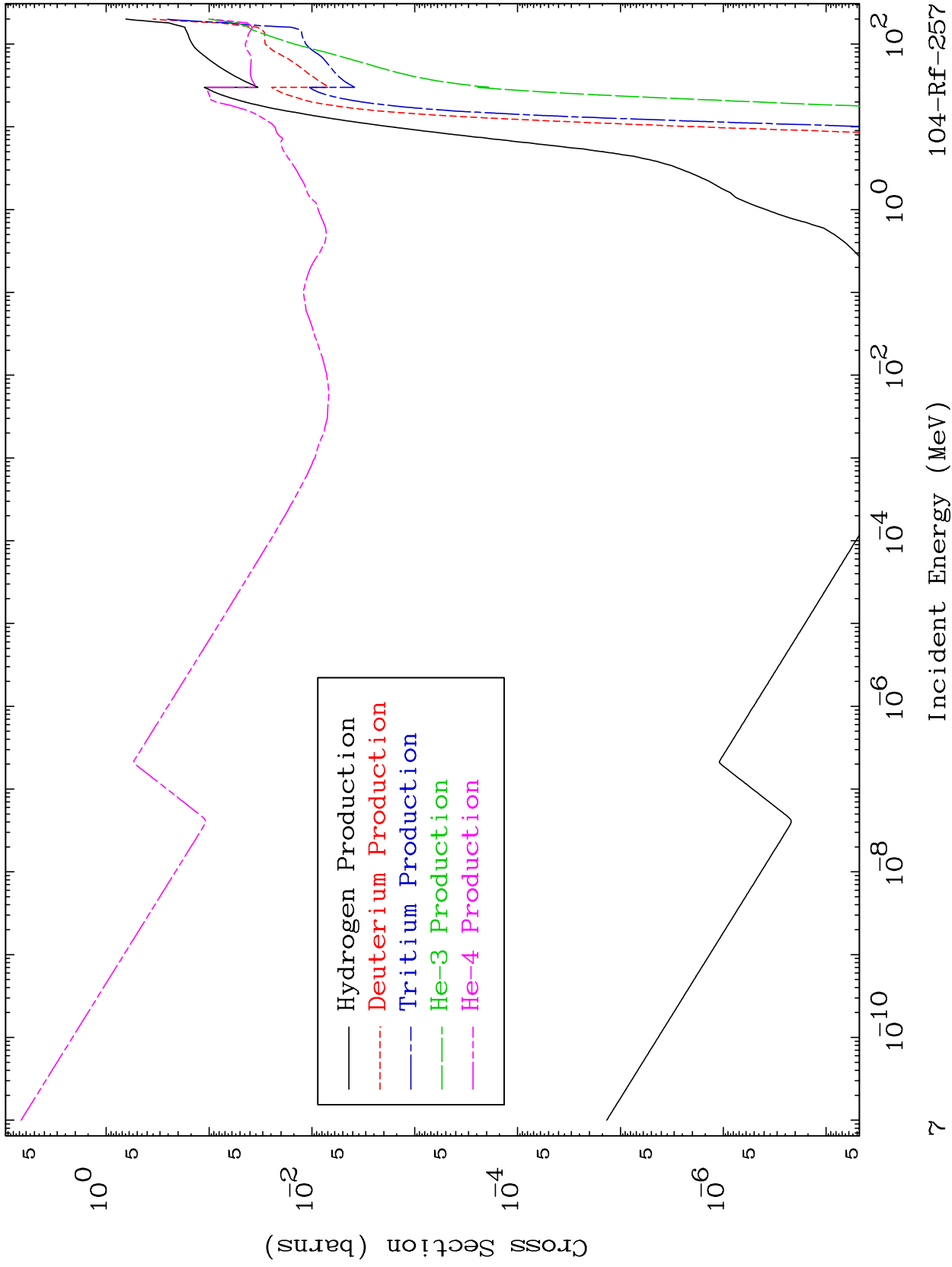
104-Rf-257



MAT 457

Particle Production
293 Kelvin Cross Sections

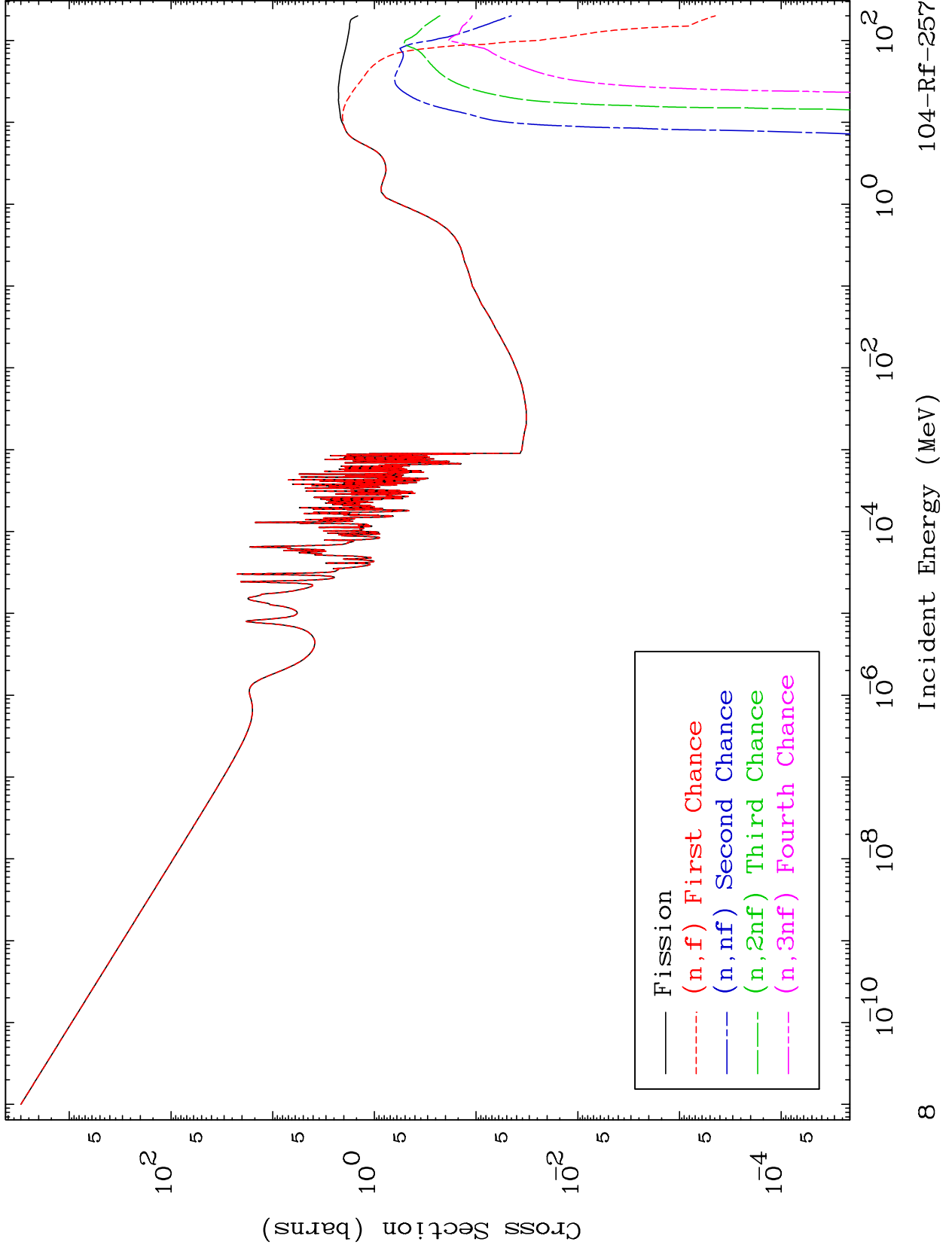
104-Rf-257



MAT 457

Fission
293 Kelvin Cross Sections

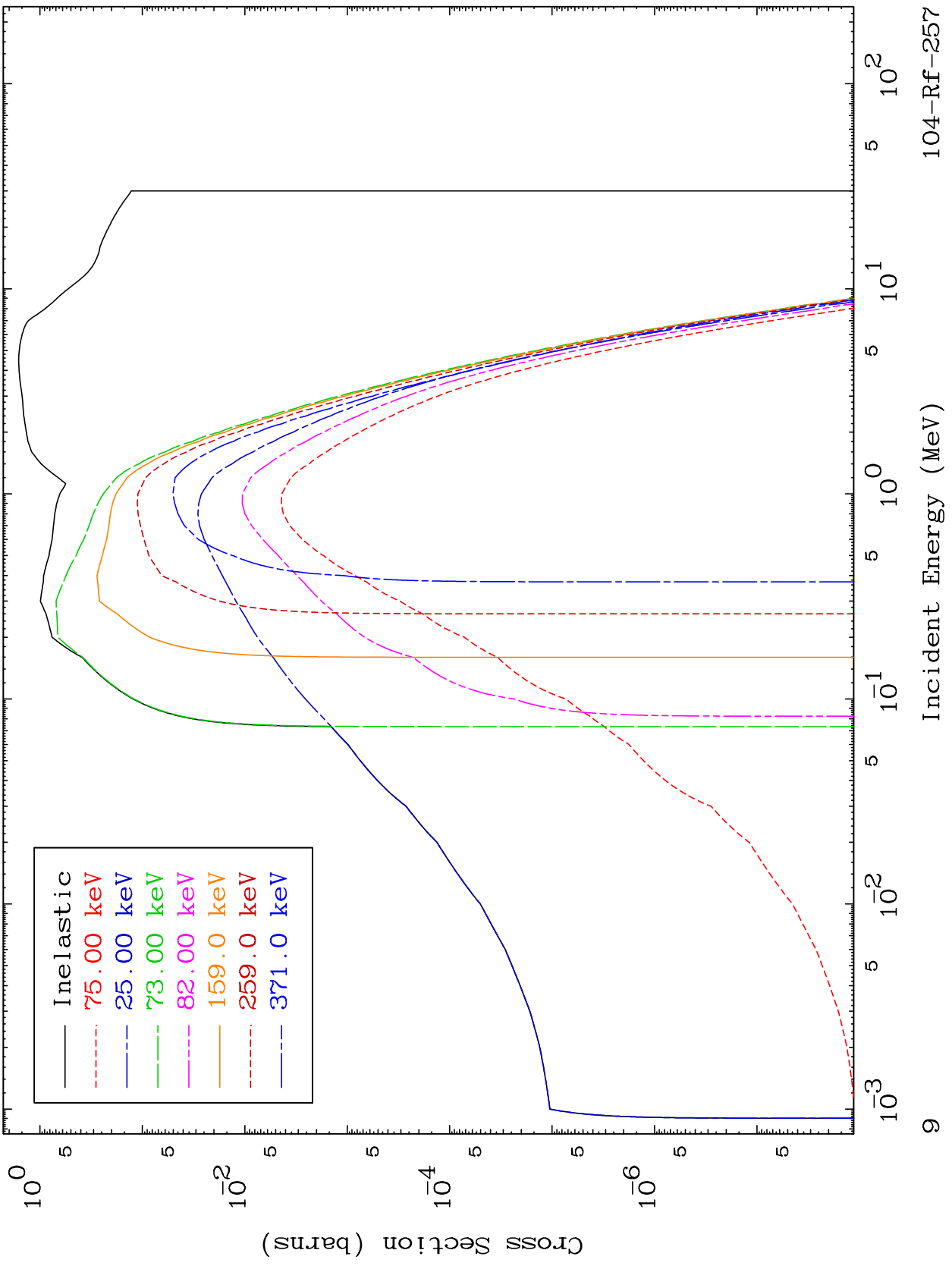
104-Rf-257



MAT 457

(n,n') Level
293 Kelvin Cross Sections

104-Rf-257



104-Rf-257

Incident Energy (MeV)

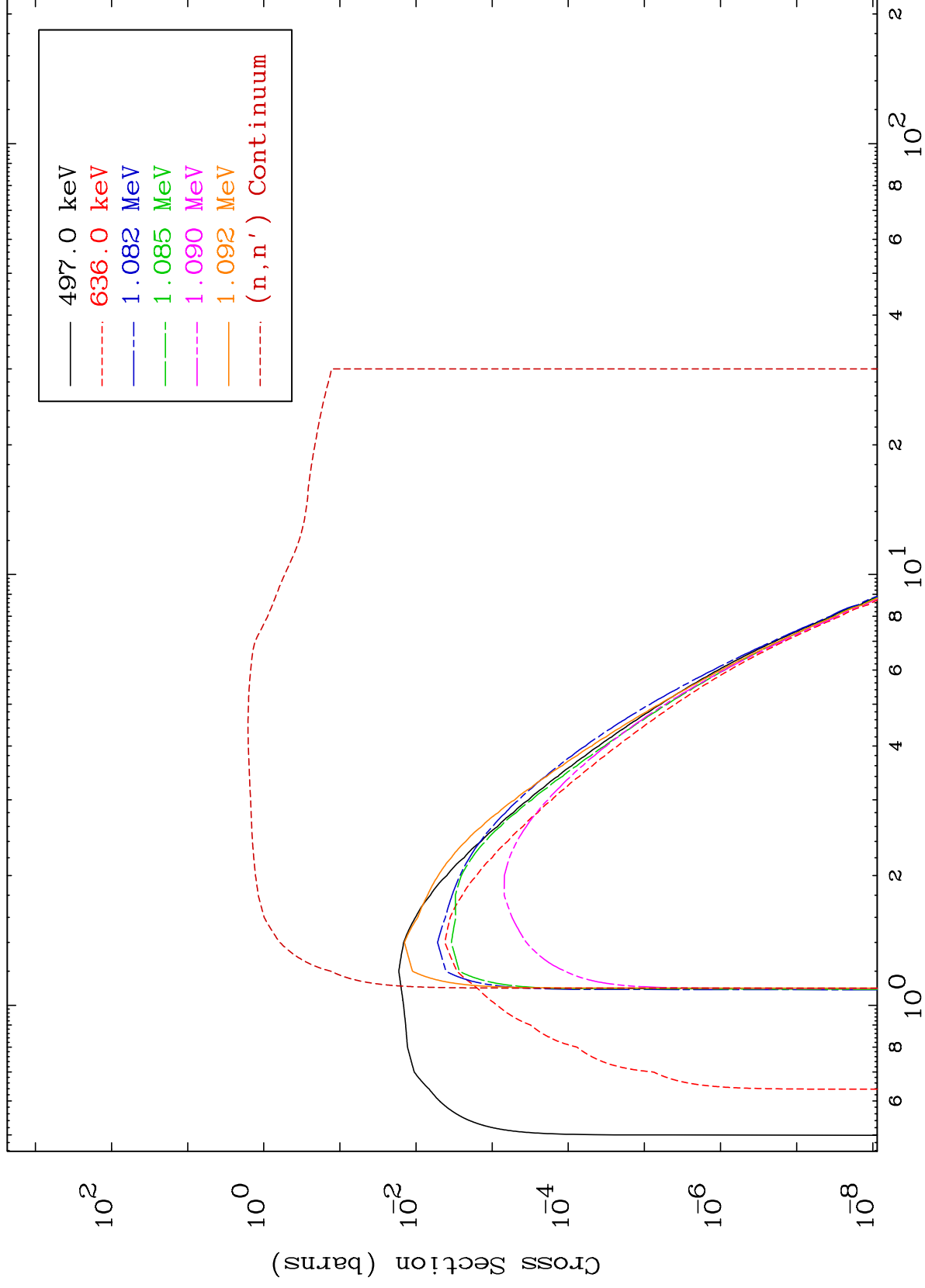
9

MAT 457

(n,n') Level

293 Kelvin Cross Sections

104-Rf-257



10

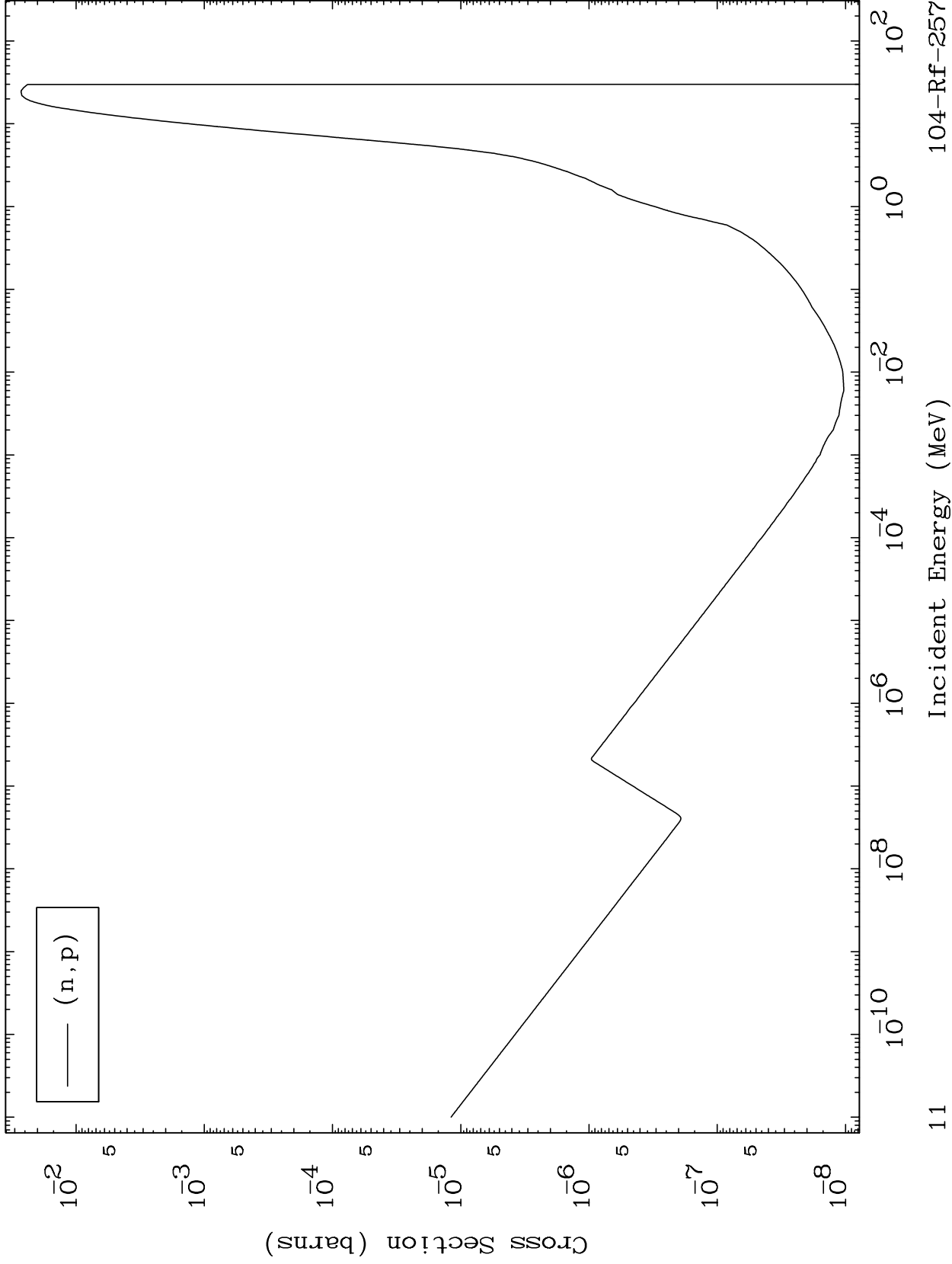
Incident Energy (MeV)

104-Rf-257

MAT 457

(n,p) Levels
293 Kelvin Cross Sections

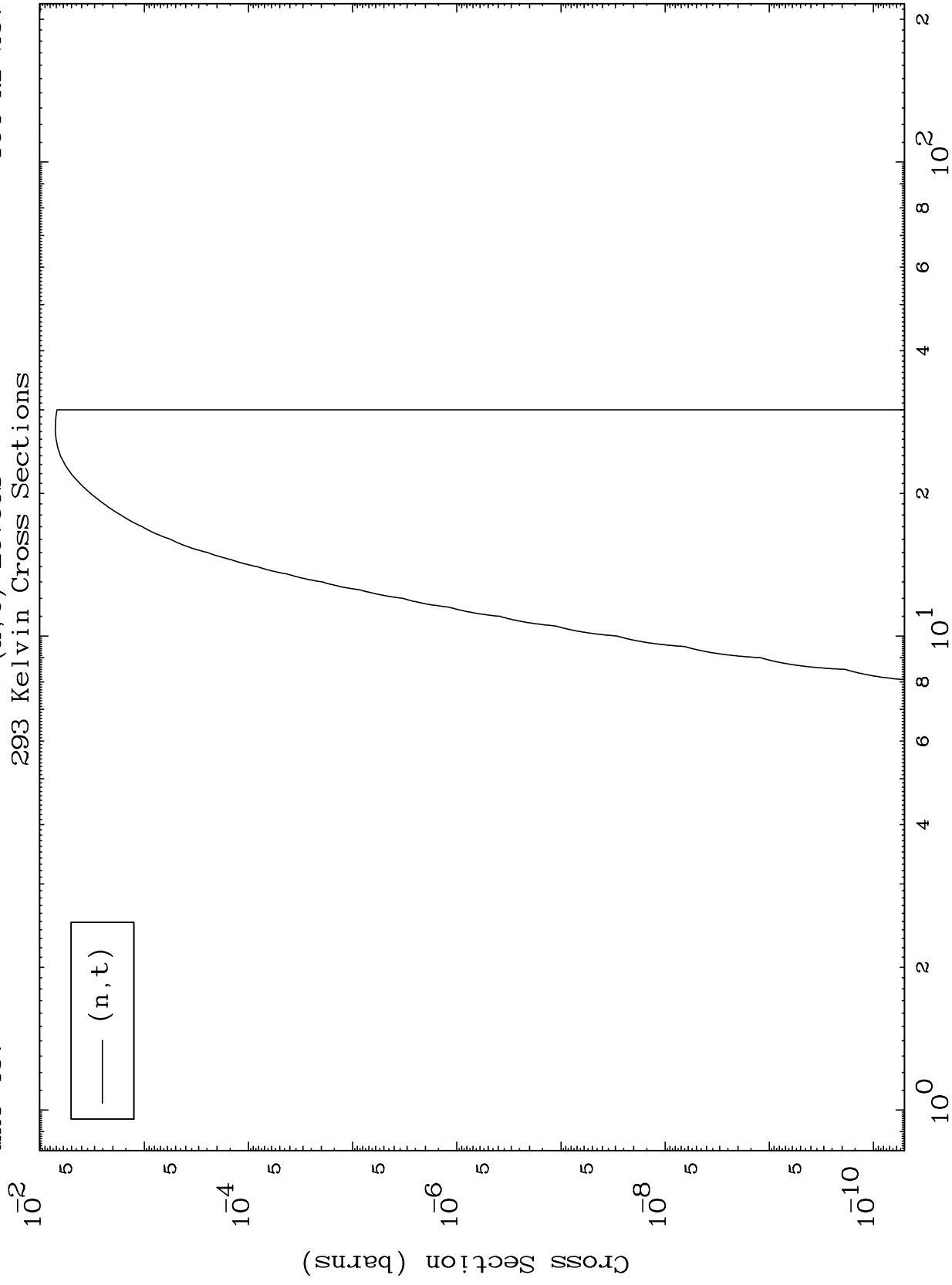
104-Rf-257



MAT 457

(n,t) Levels
293 Kelvin Cross Sections

104-Rf-257



13

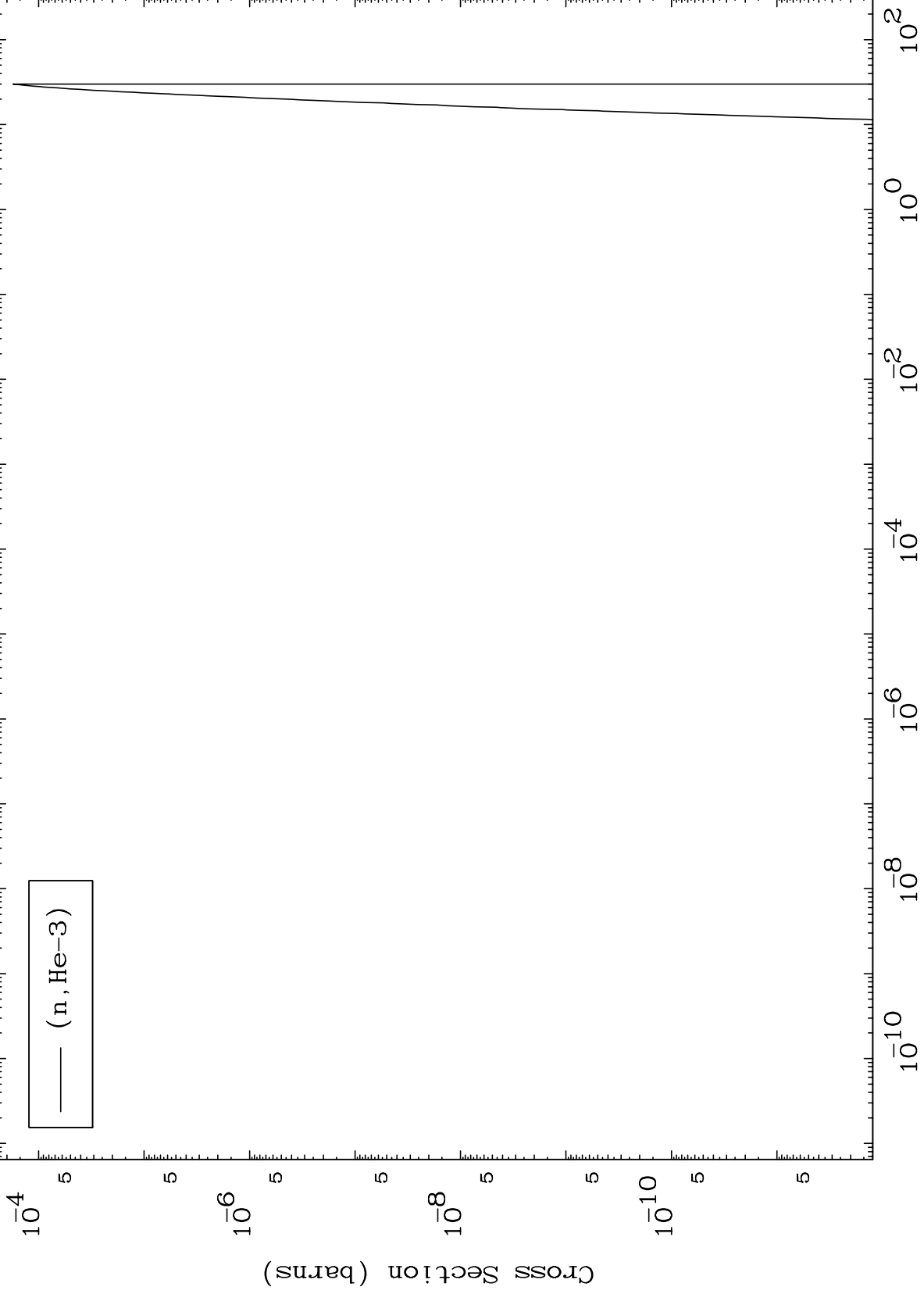
Incident Energy (MeV)

104-Rf-257

MAT 457

(n,He3) Levels
293 Kelvin Cross Sections

104-Rf-257

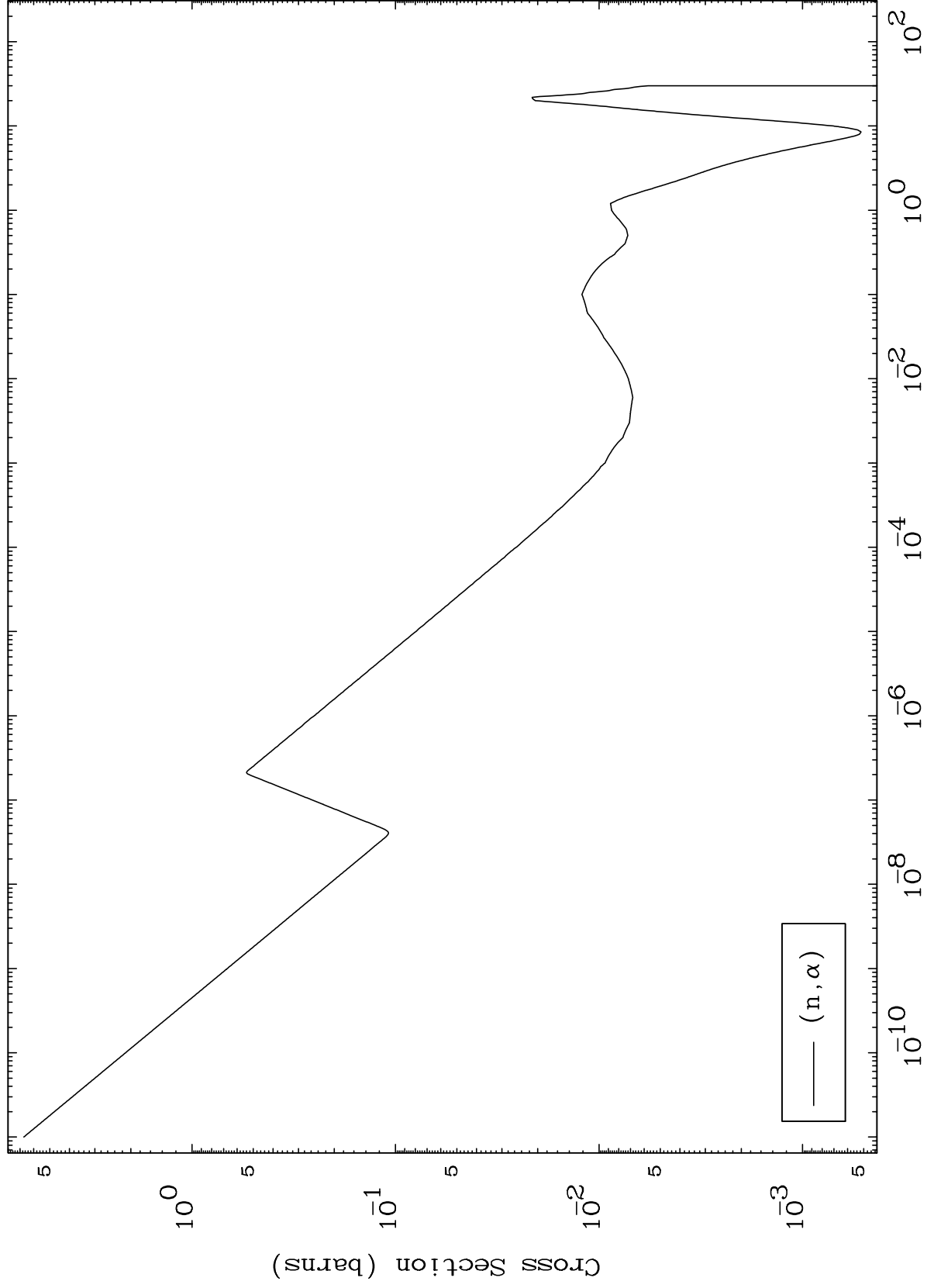


(n, He-3)

MAT 457

(n,α) Levels
293 Kelvin Cross Sections

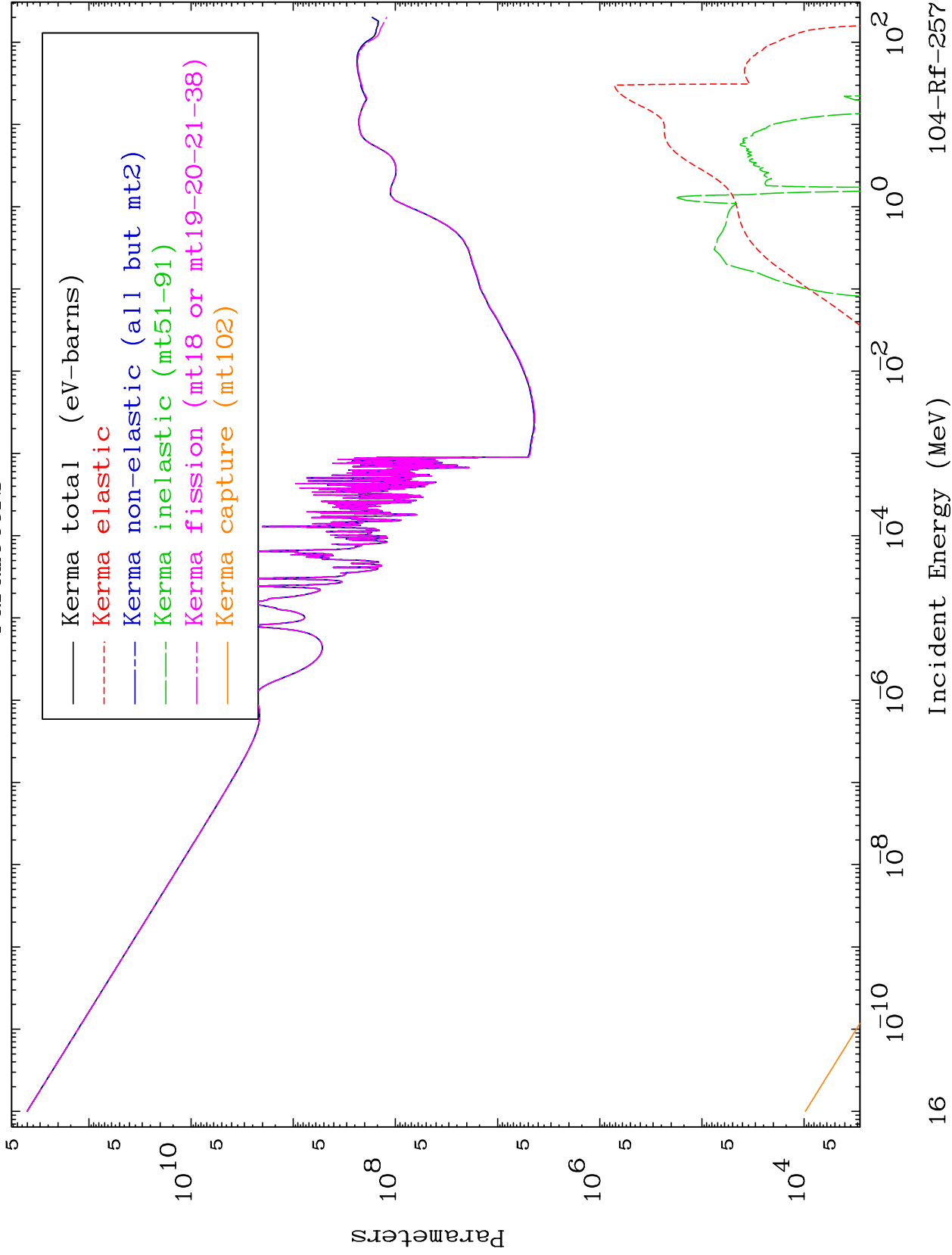
104-Rf-257

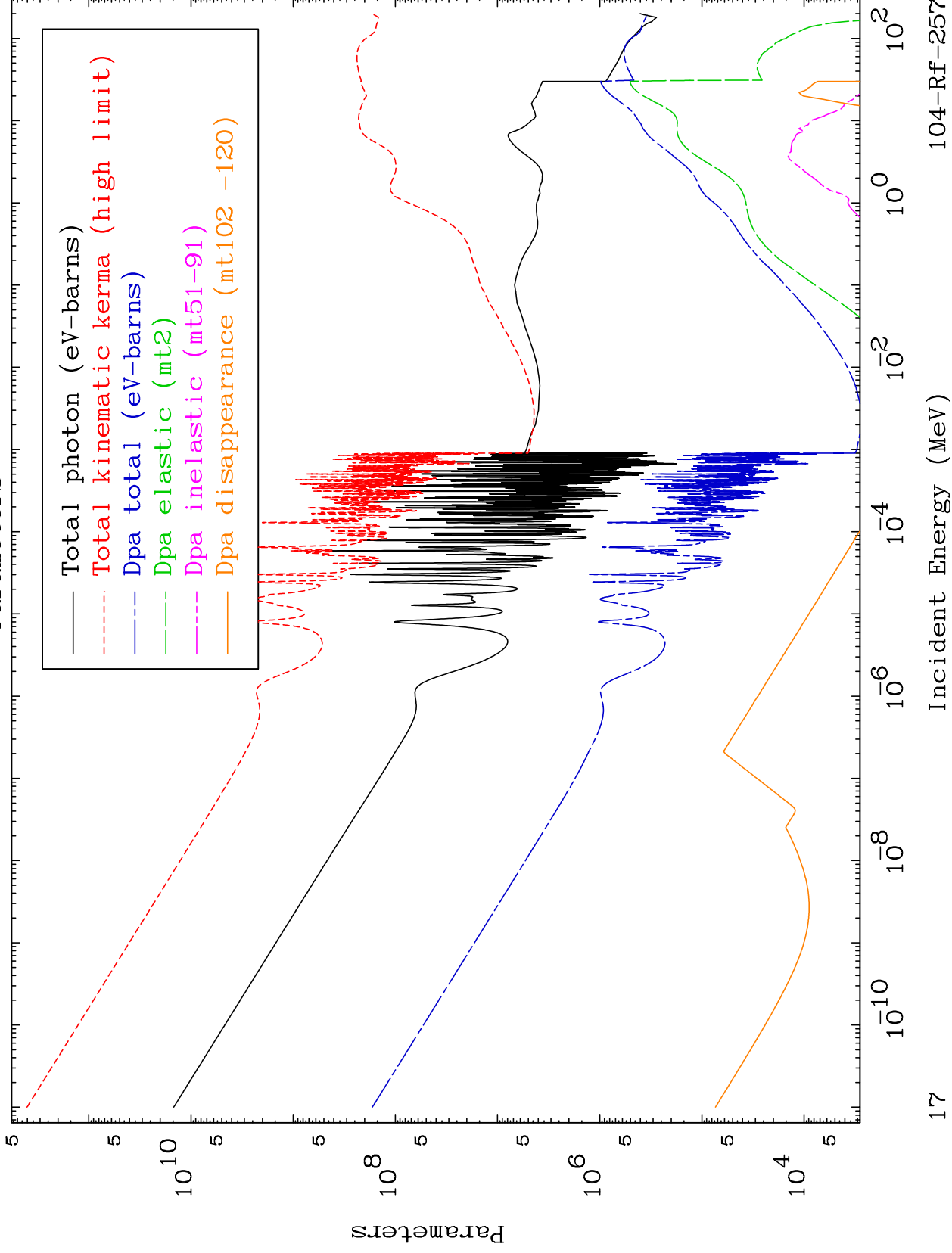


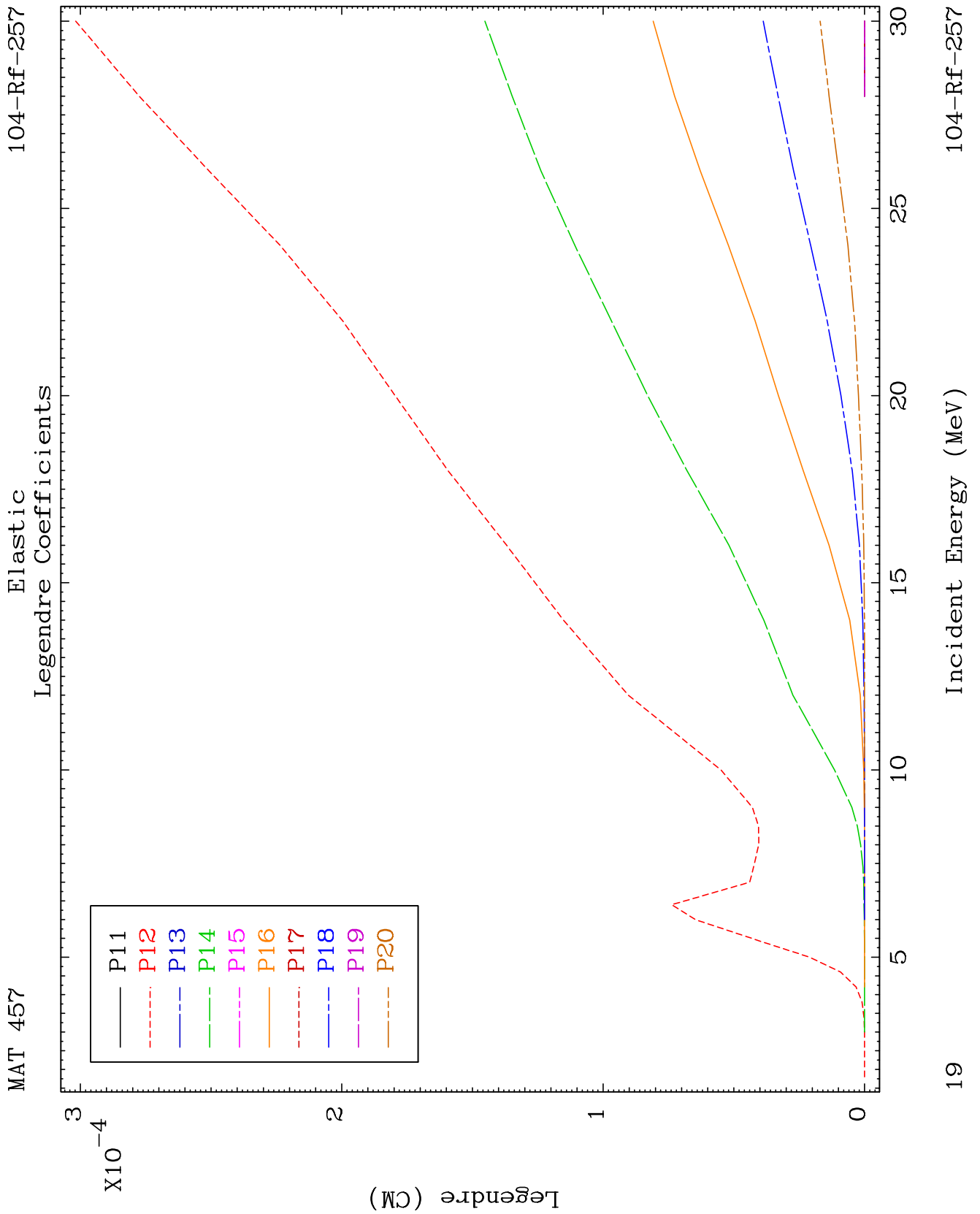
15

Incident Energy (MeV)

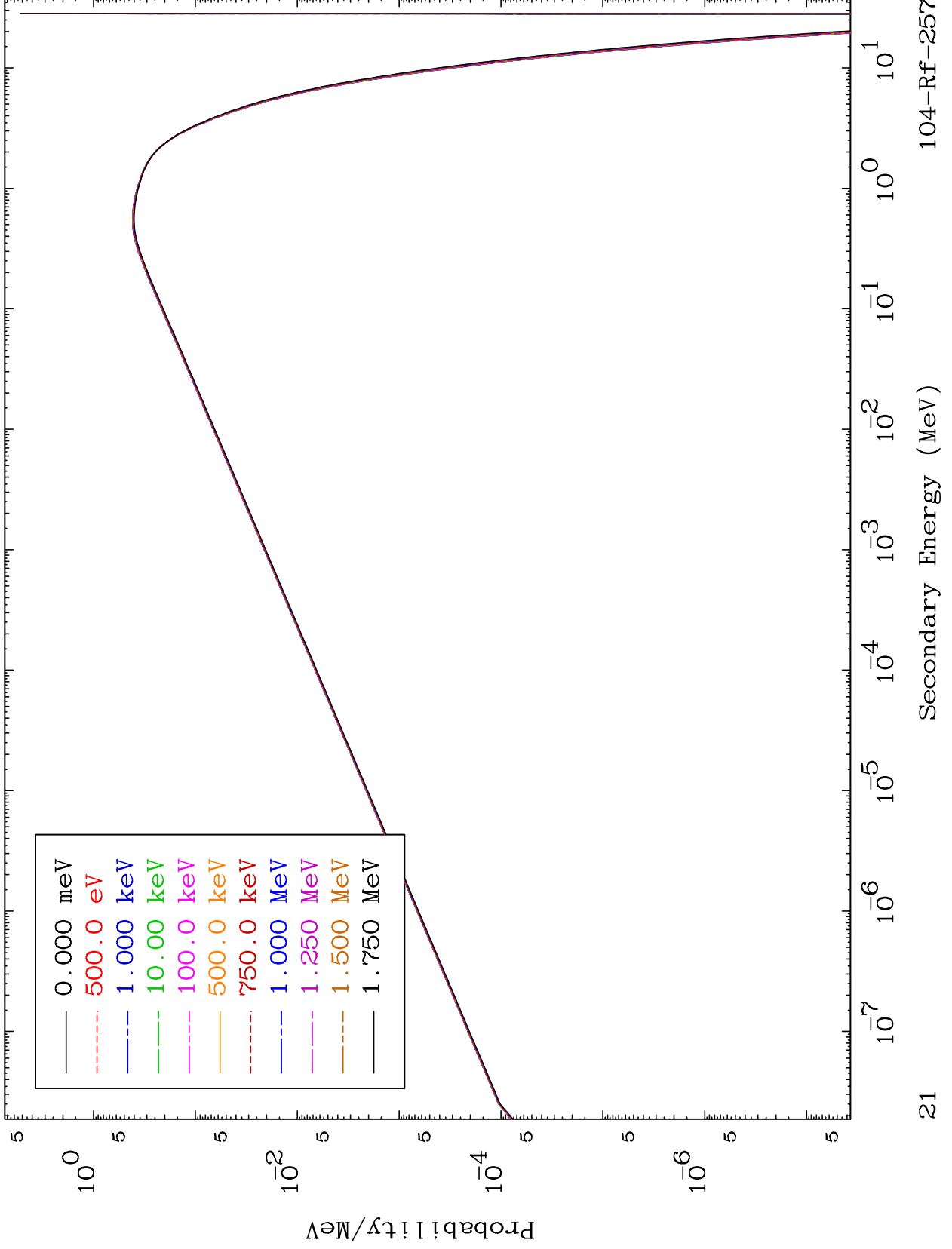
104-Rf-257



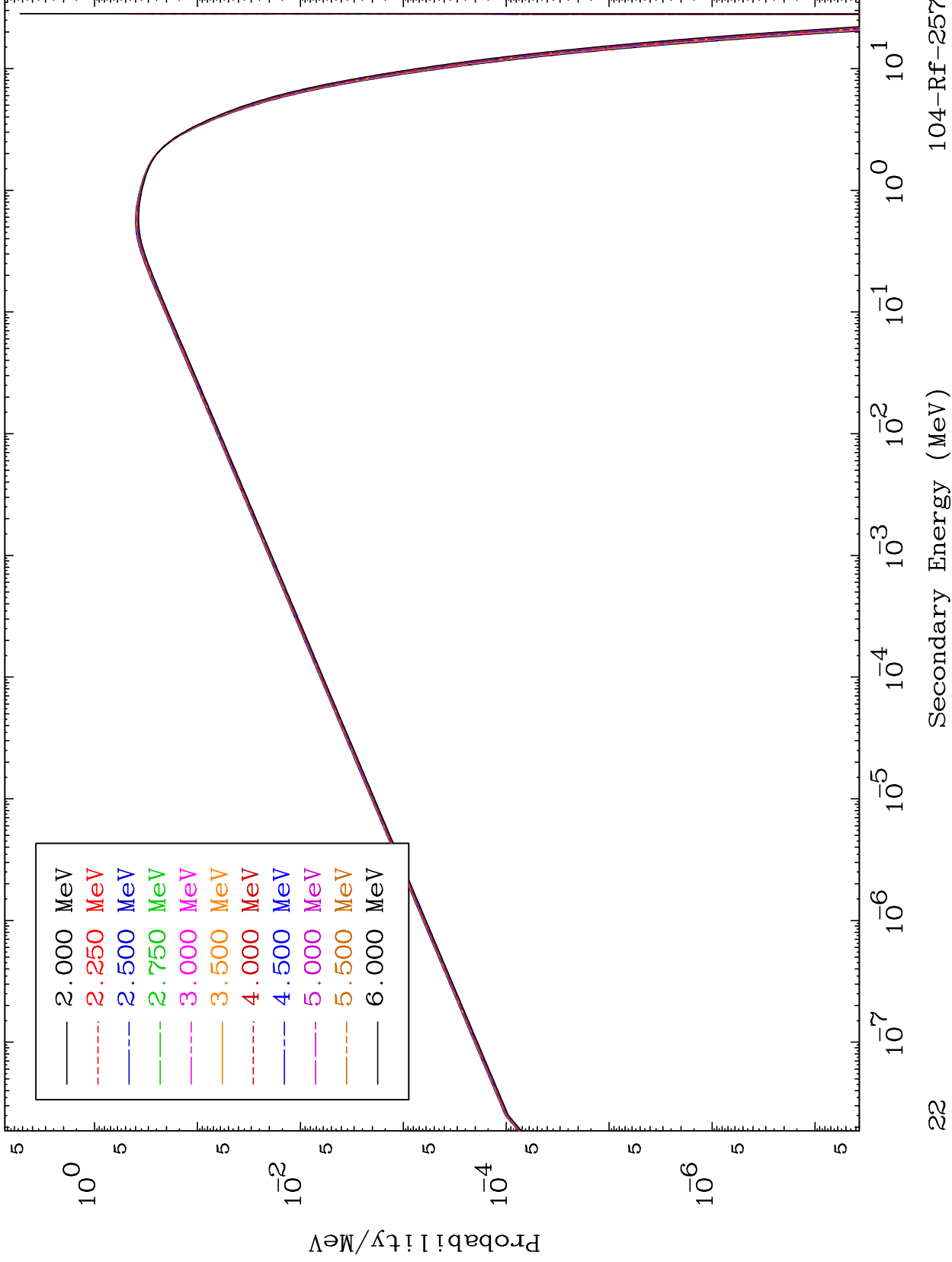




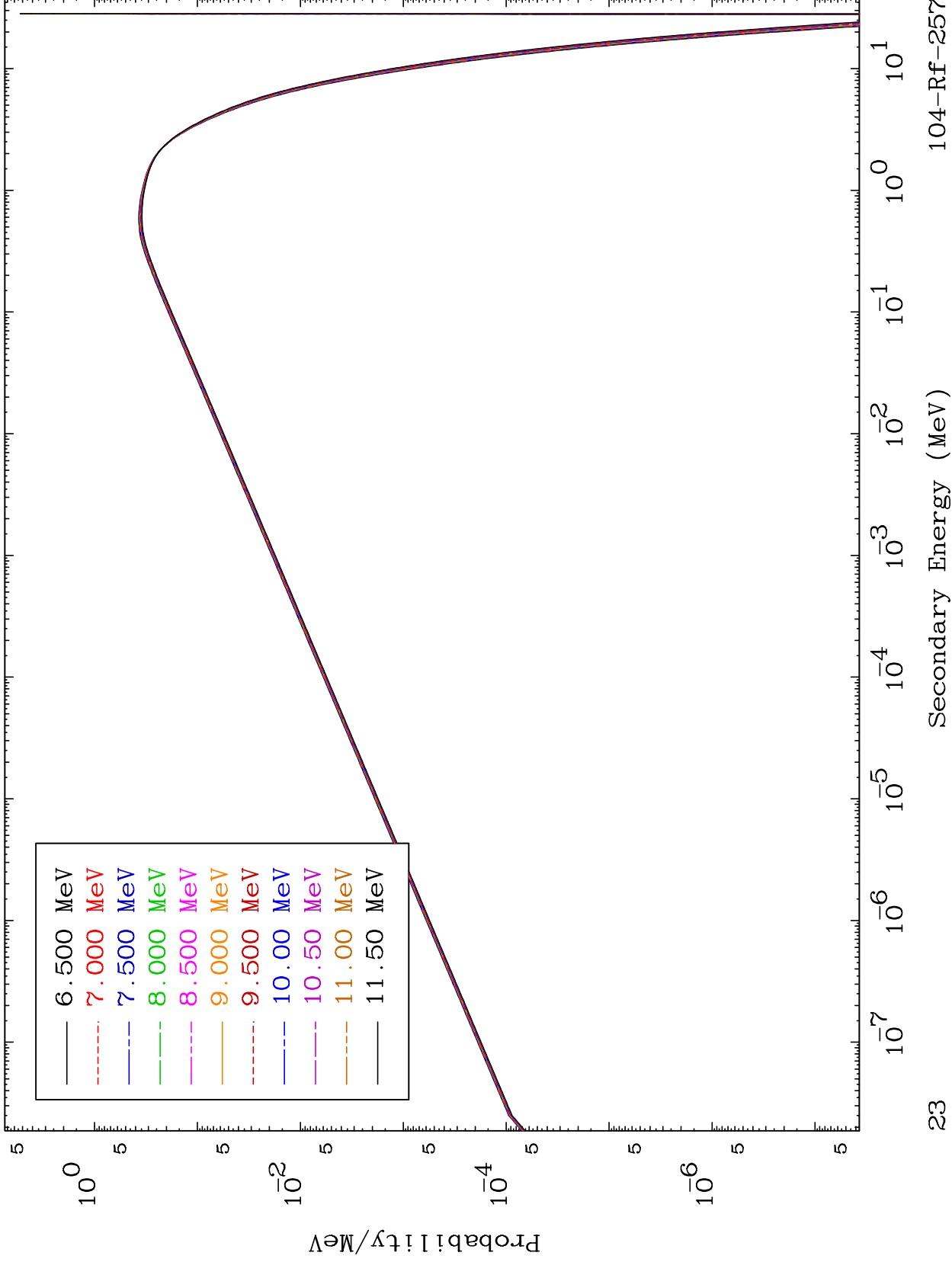
Fission
Energy Distribution

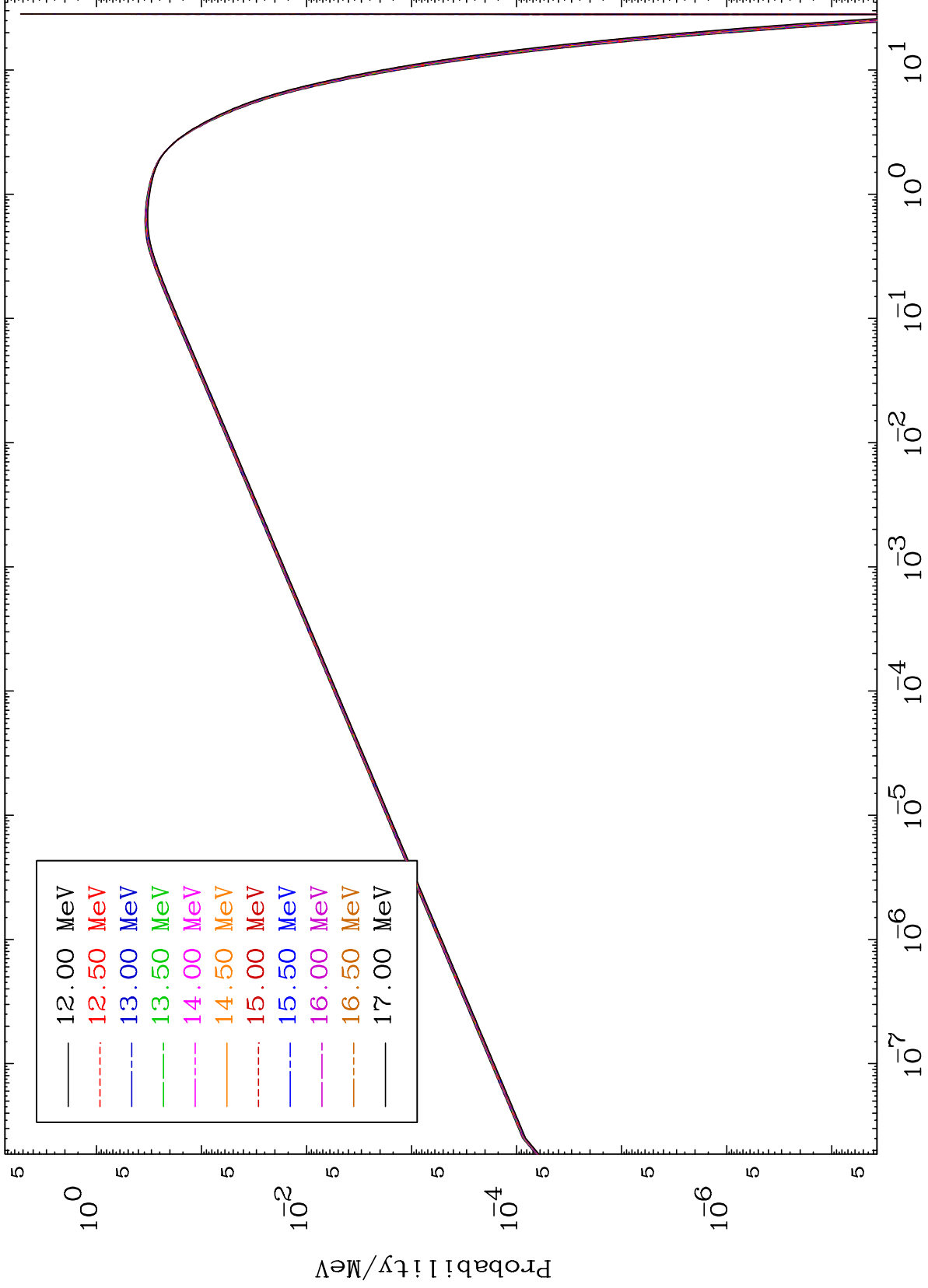


Fission
Energy Distribution

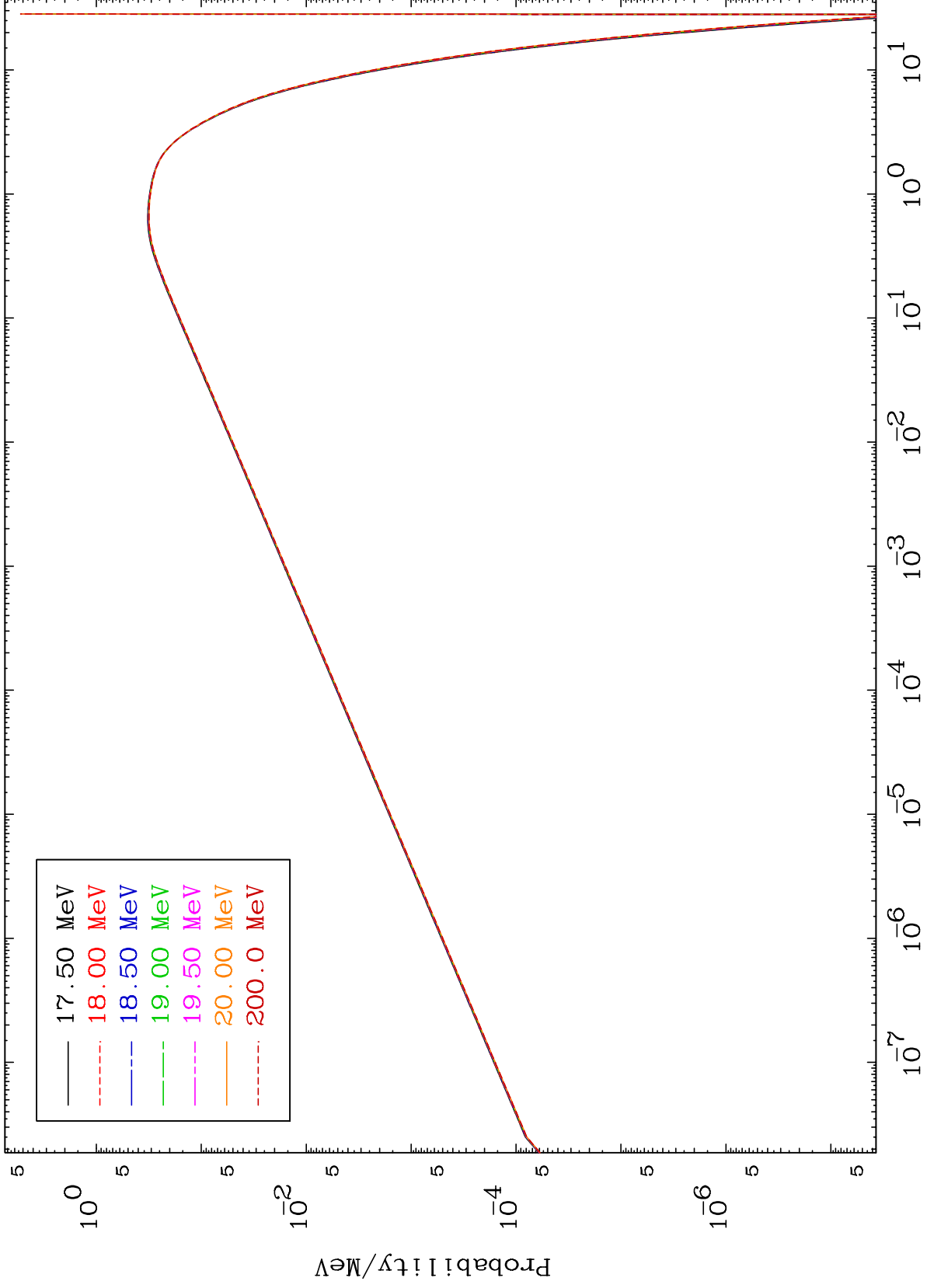


Fission
Energy Distribution





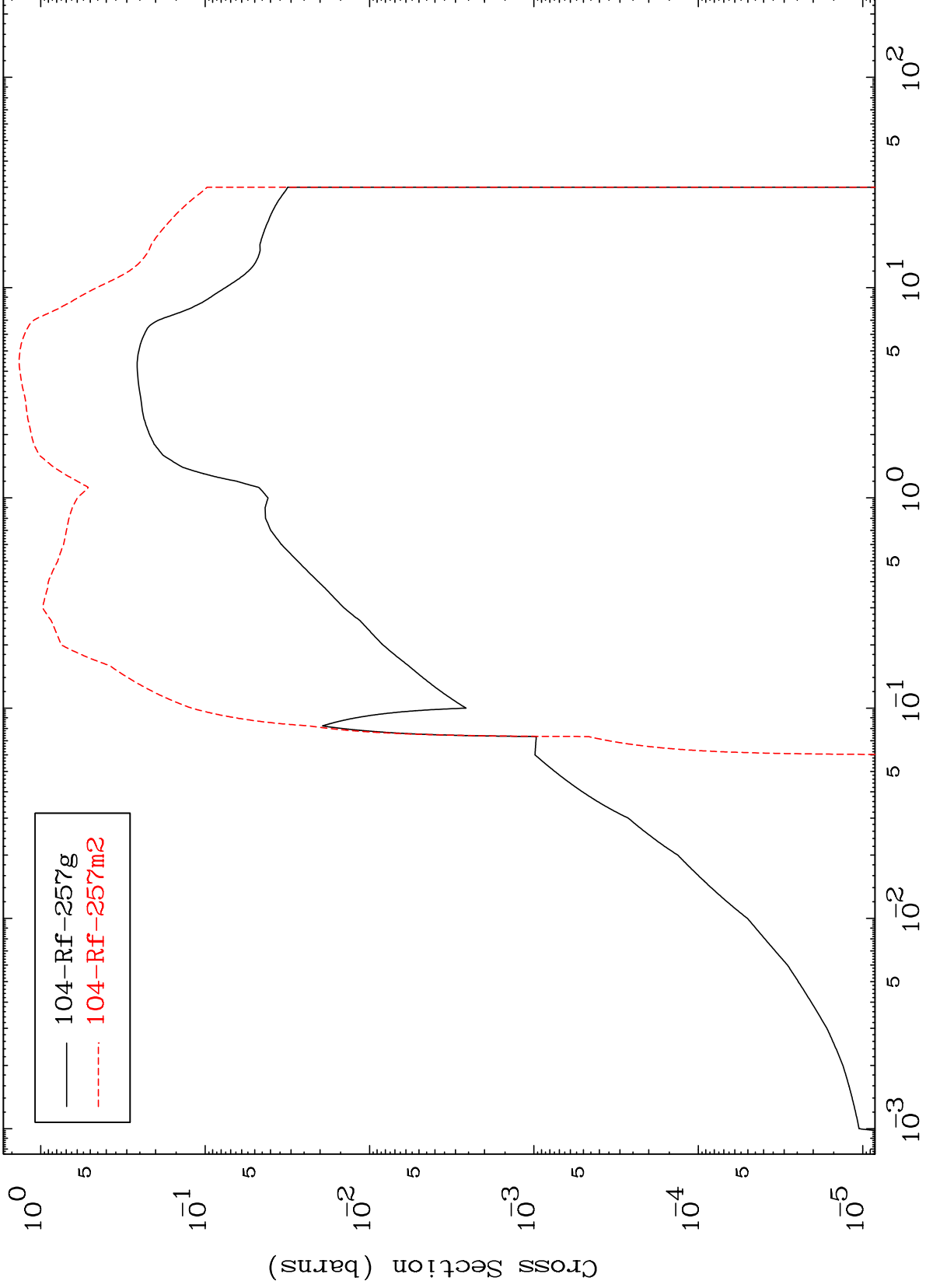
Fission
Energy Distribution



MAT 457

104-Rf-257

Inelastic
Radionuclide Production Cross Section



26

Incident Energy (MeV)

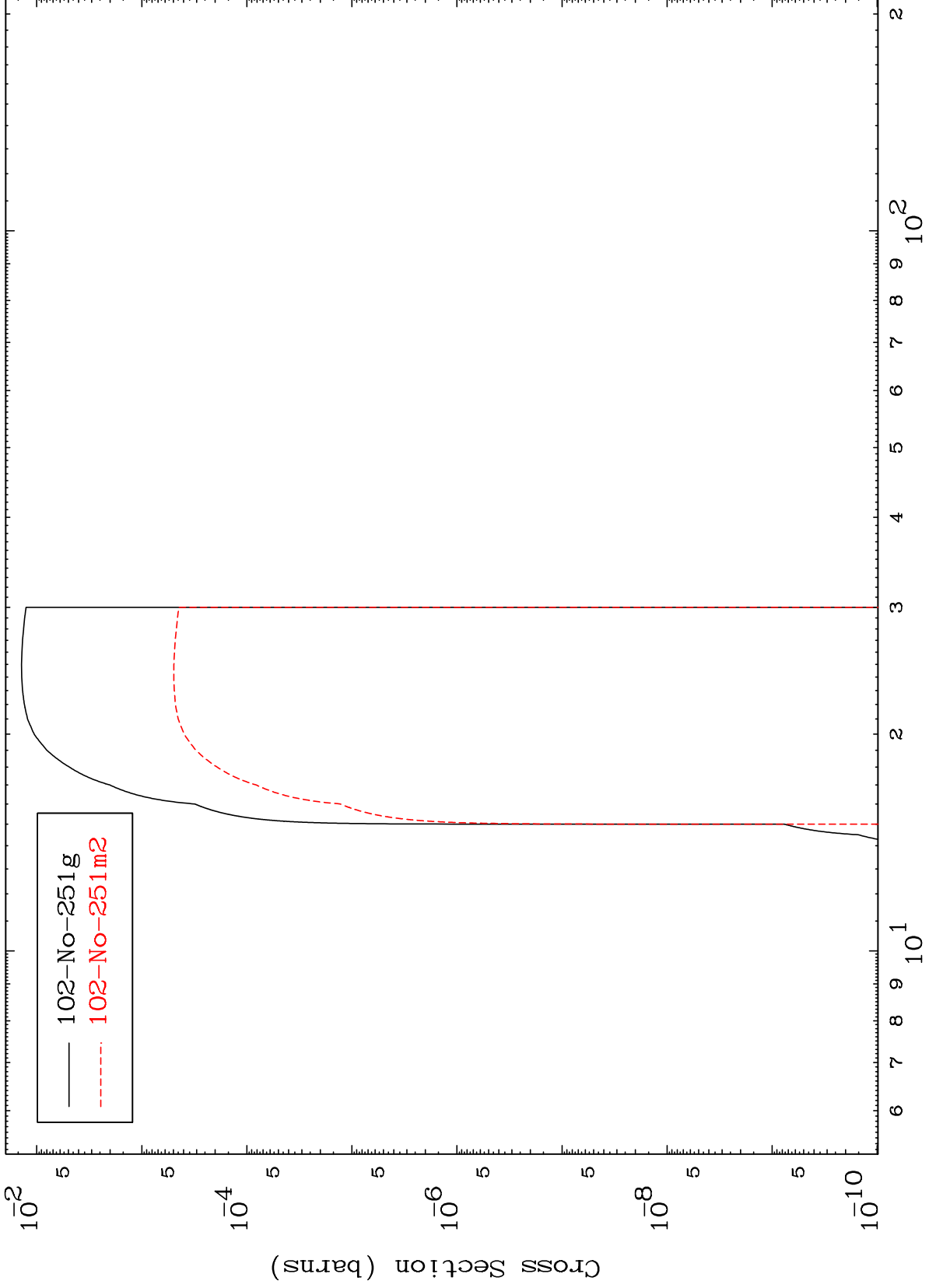
104-Rf-257

MAT 457

(n,3n) α

104-Rf-257

Radionuclide Production Cross Section



27

Incident Energy (MeV)

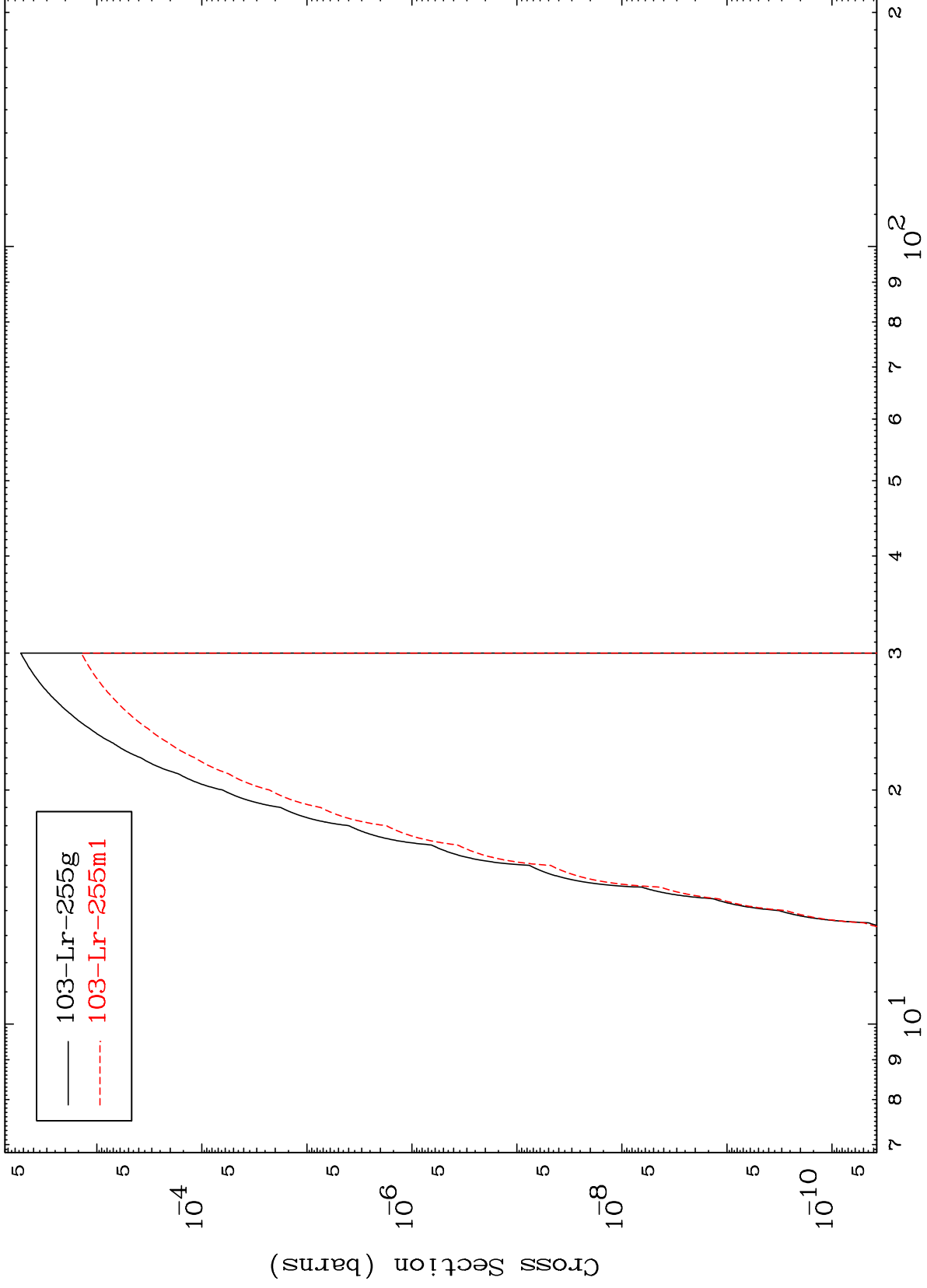
104-Rf-257

MAT 457

(n,n') d

104-Rf-257

Radionuclide Production Cross Section



28

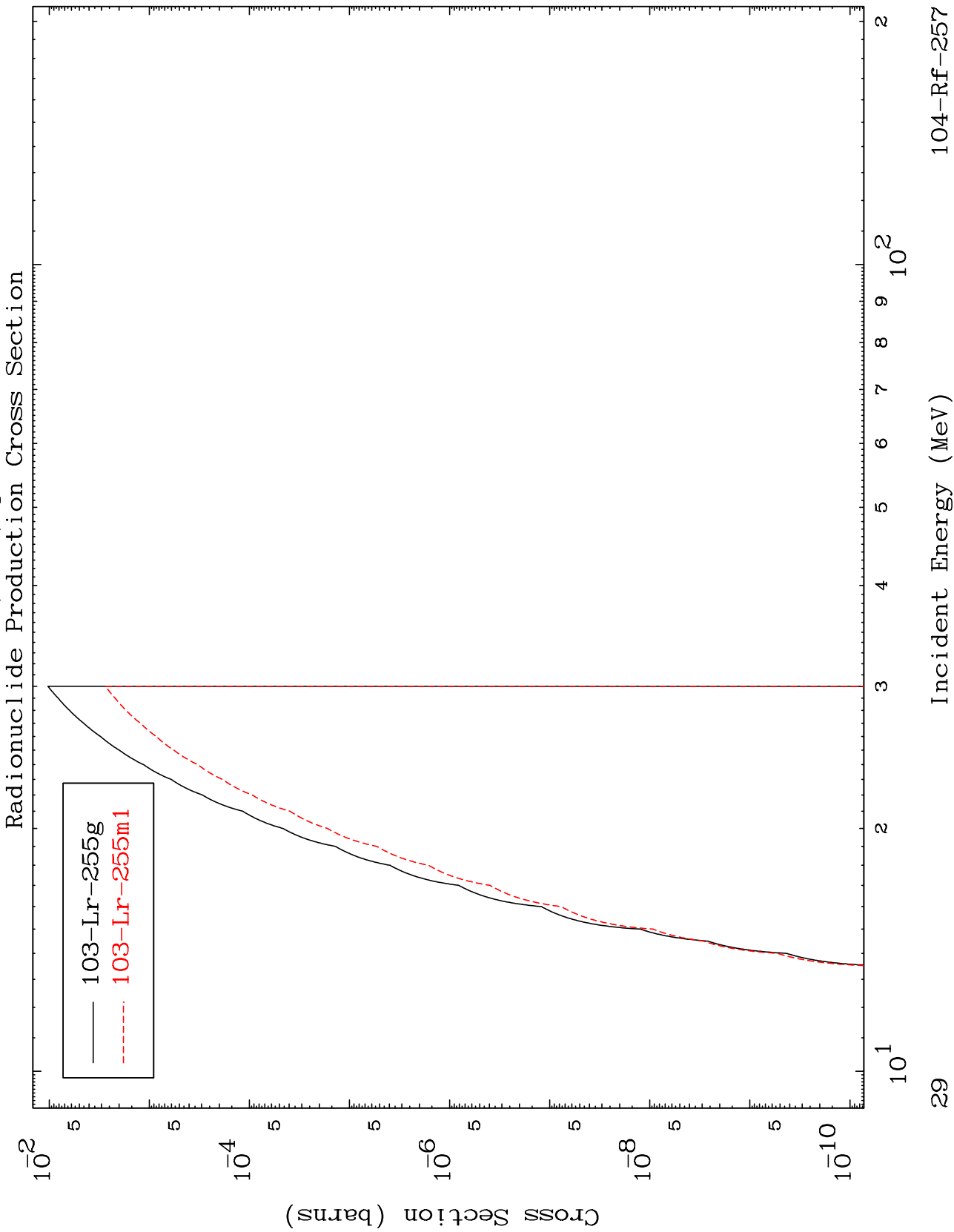
Incident Energy (MeV)

104-Rf-257

MAT 457

(n,2n) p

104-Rf-257



29

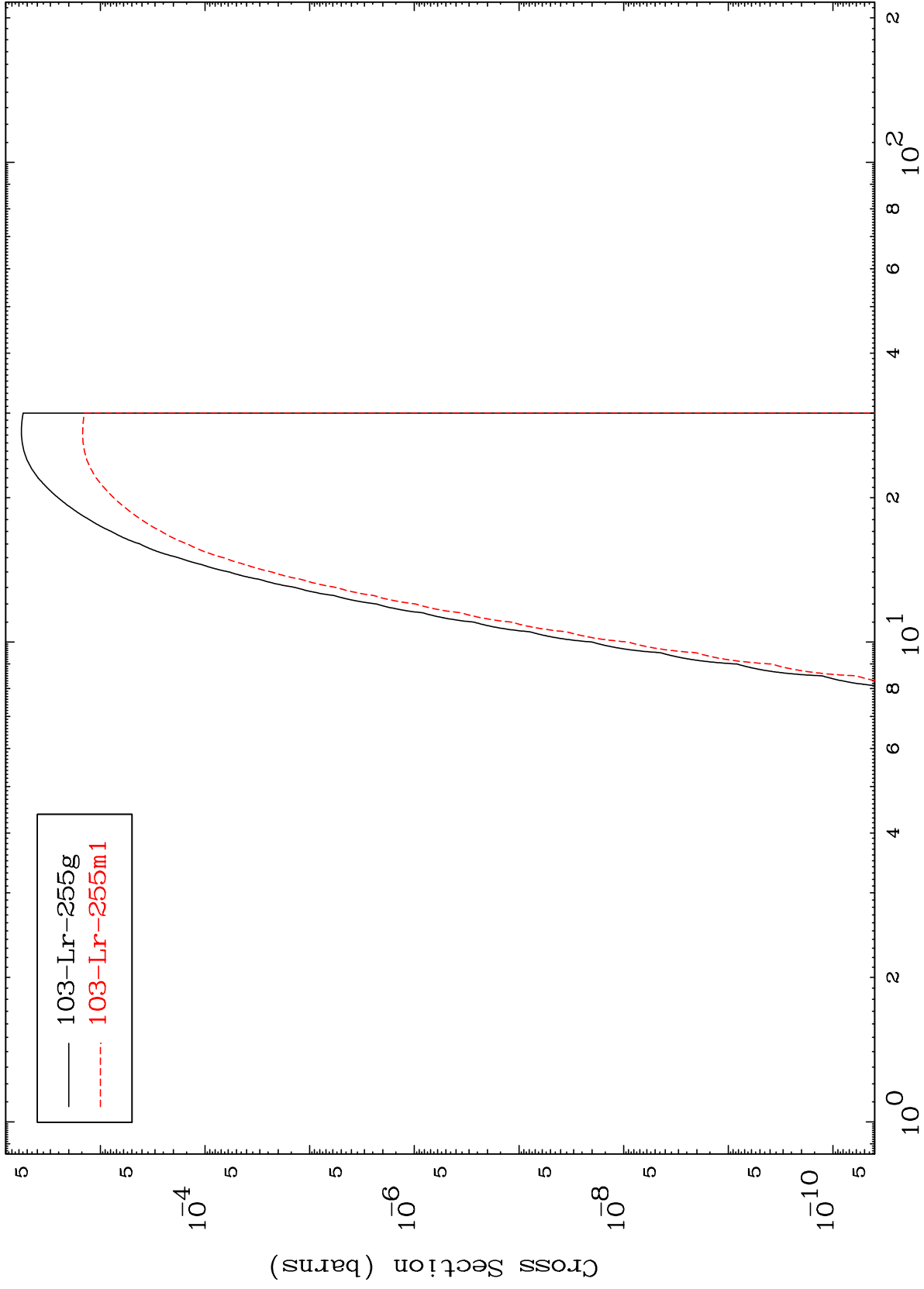
Incident Energy (MeV)

104-Rf-257

MAT 457

104-Rf-257

(n, t)
Radionuclide Production Cross Section



104-Rf-257

Incident Energy (MeV)

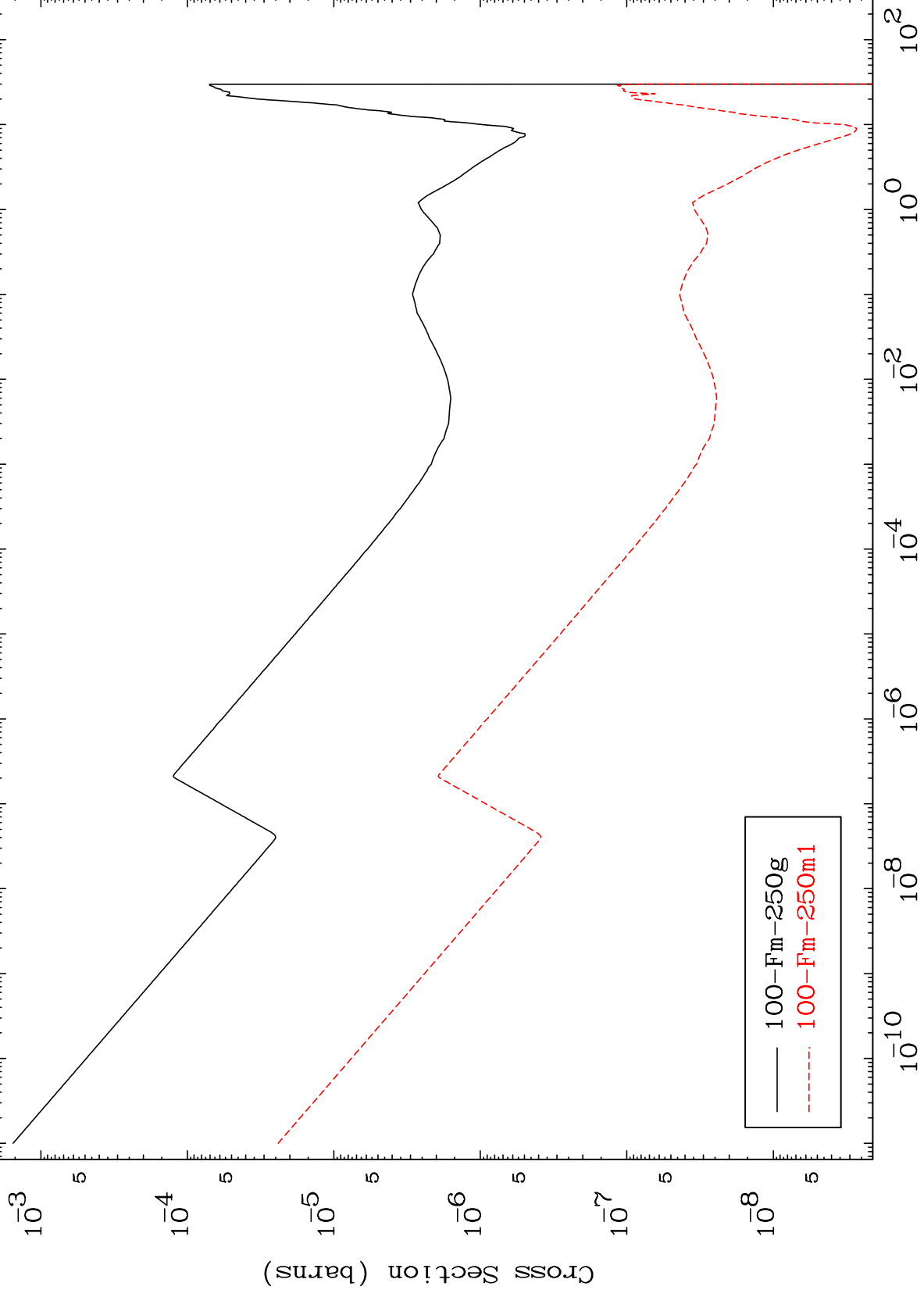
30

MAT 457

(n,2α)

104-Rf-257

Radionuclide Production Cross Section



— 100-Fm-250g
- - - 100-Fm-250m1