

Program EVALPLOT
(Version 2015-2)

by

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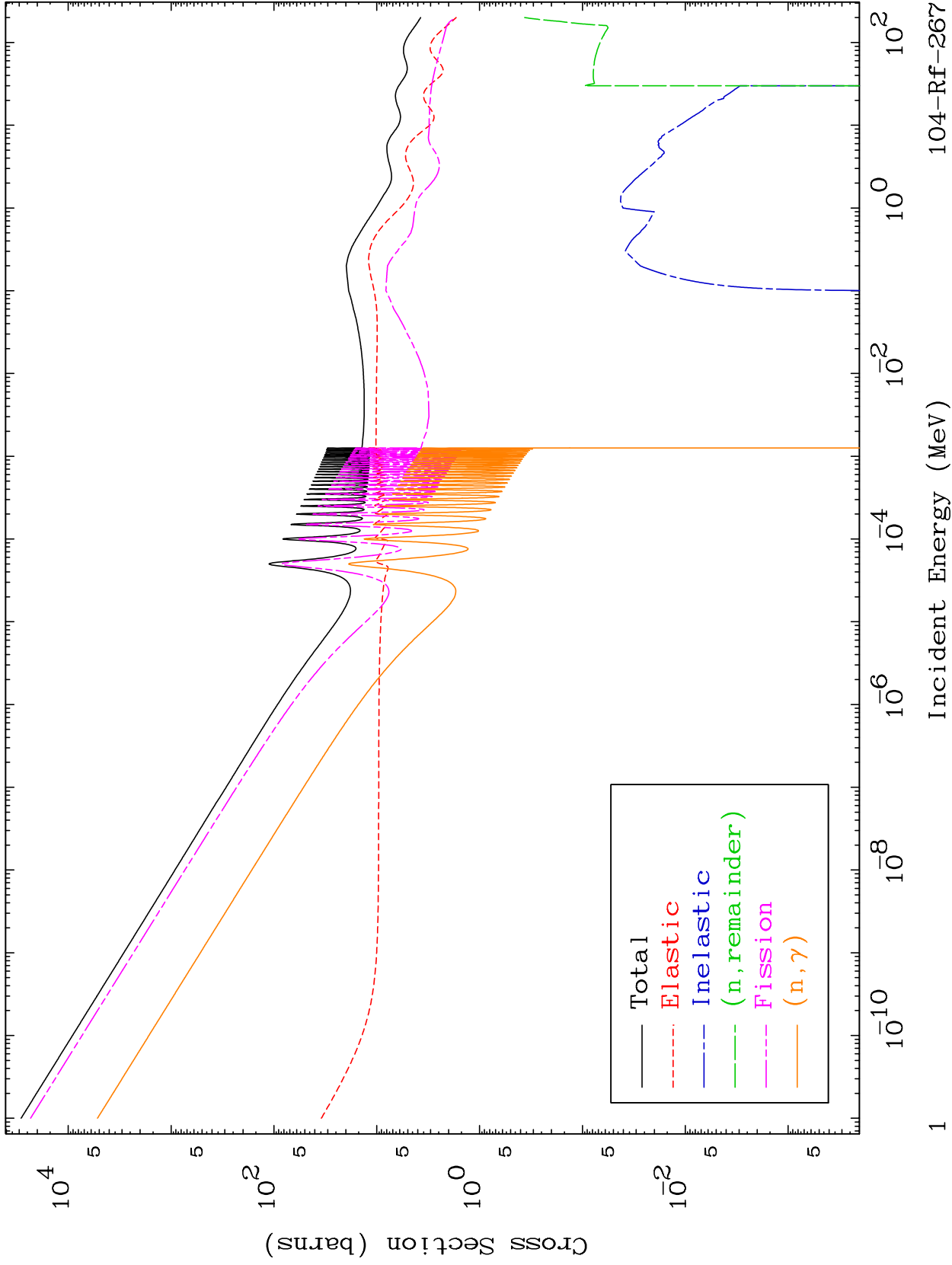
Press Mouse Button to Start

MAT 467

Major

293 Kelvin Cross Sections

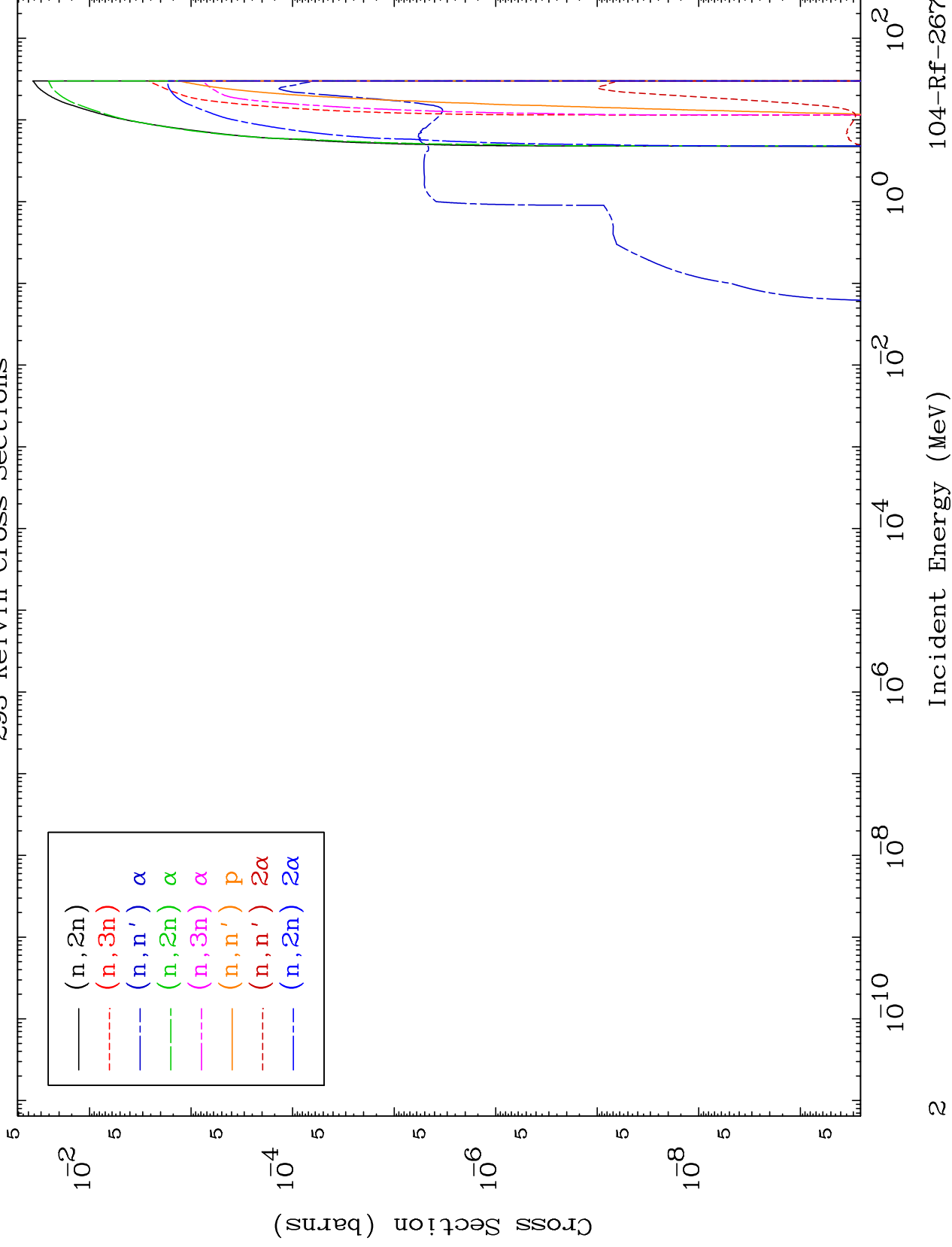
104-Rf-267



MAT 467

Neutron Production
293 Kelvin Cross Sections

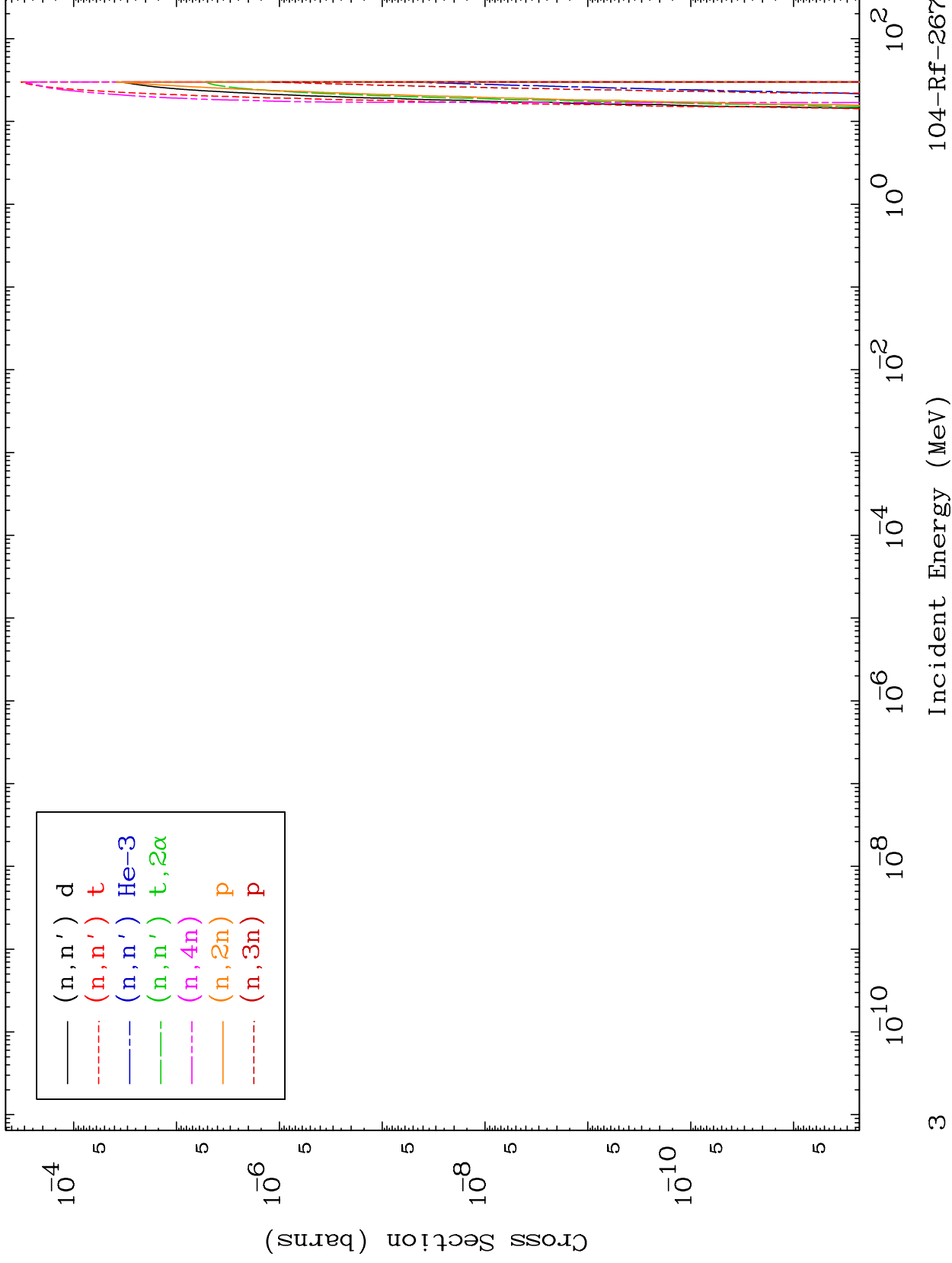
104-Rf-267



MAT 467

Neutron Production
293 Kelvin Cross Sections

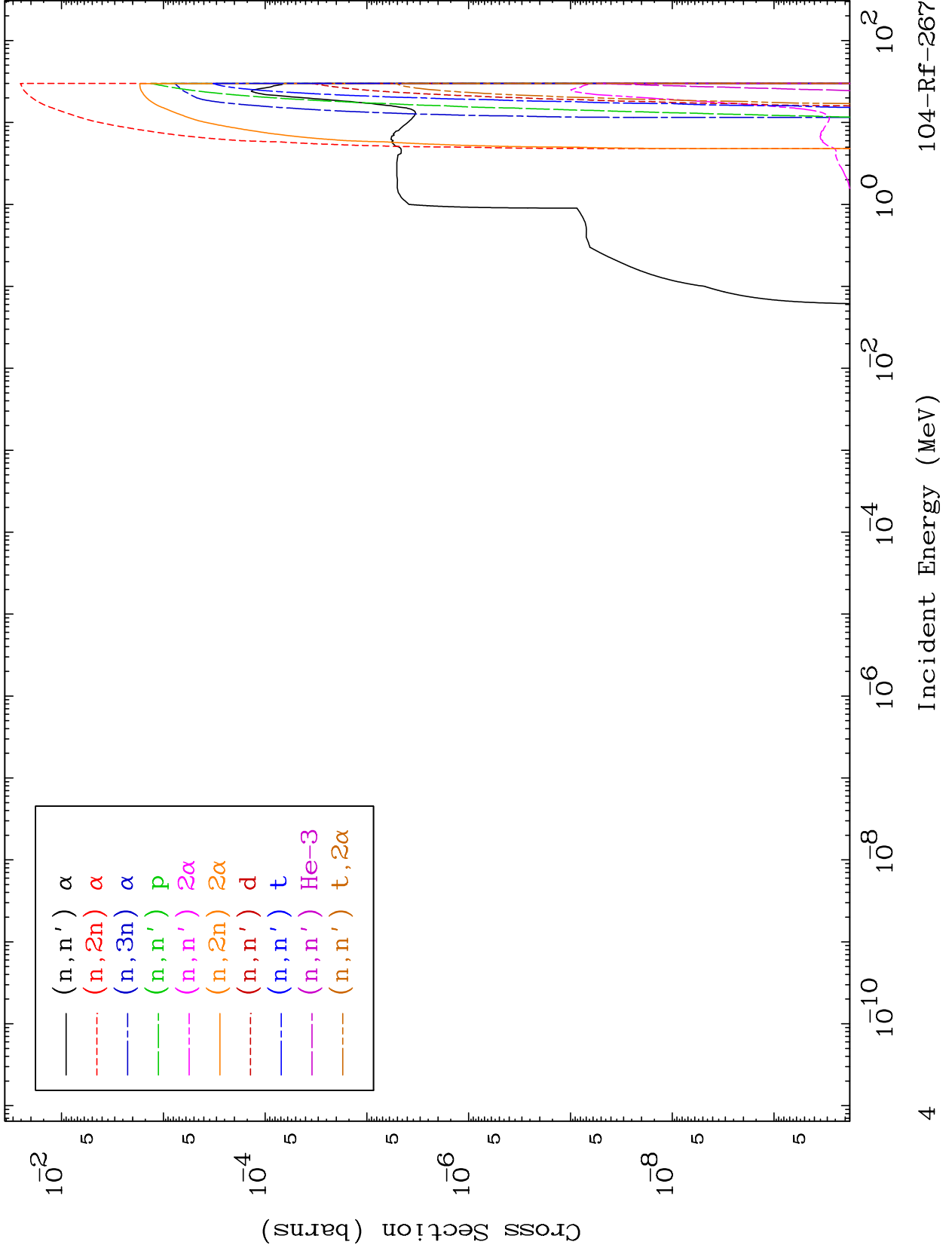
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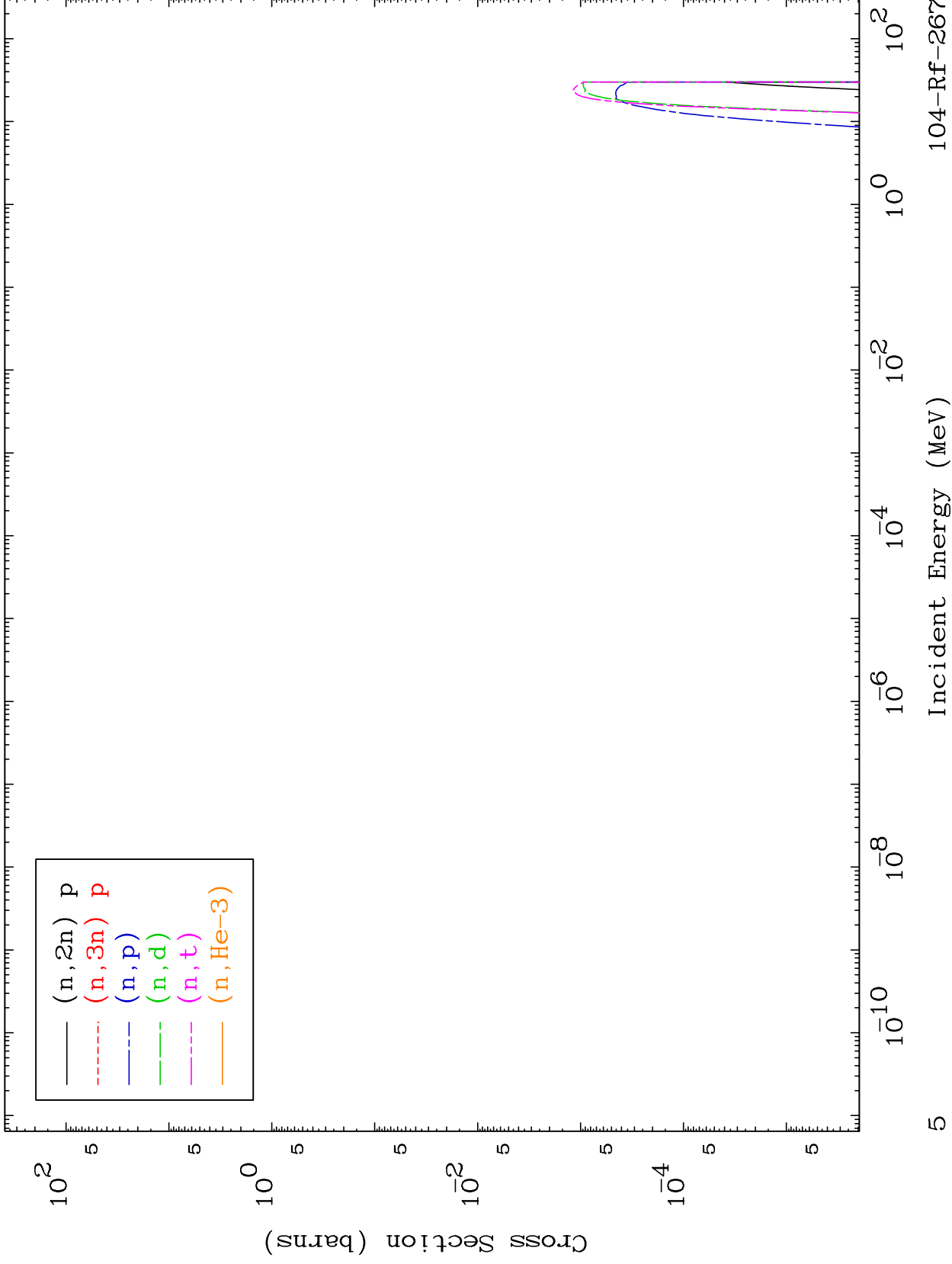


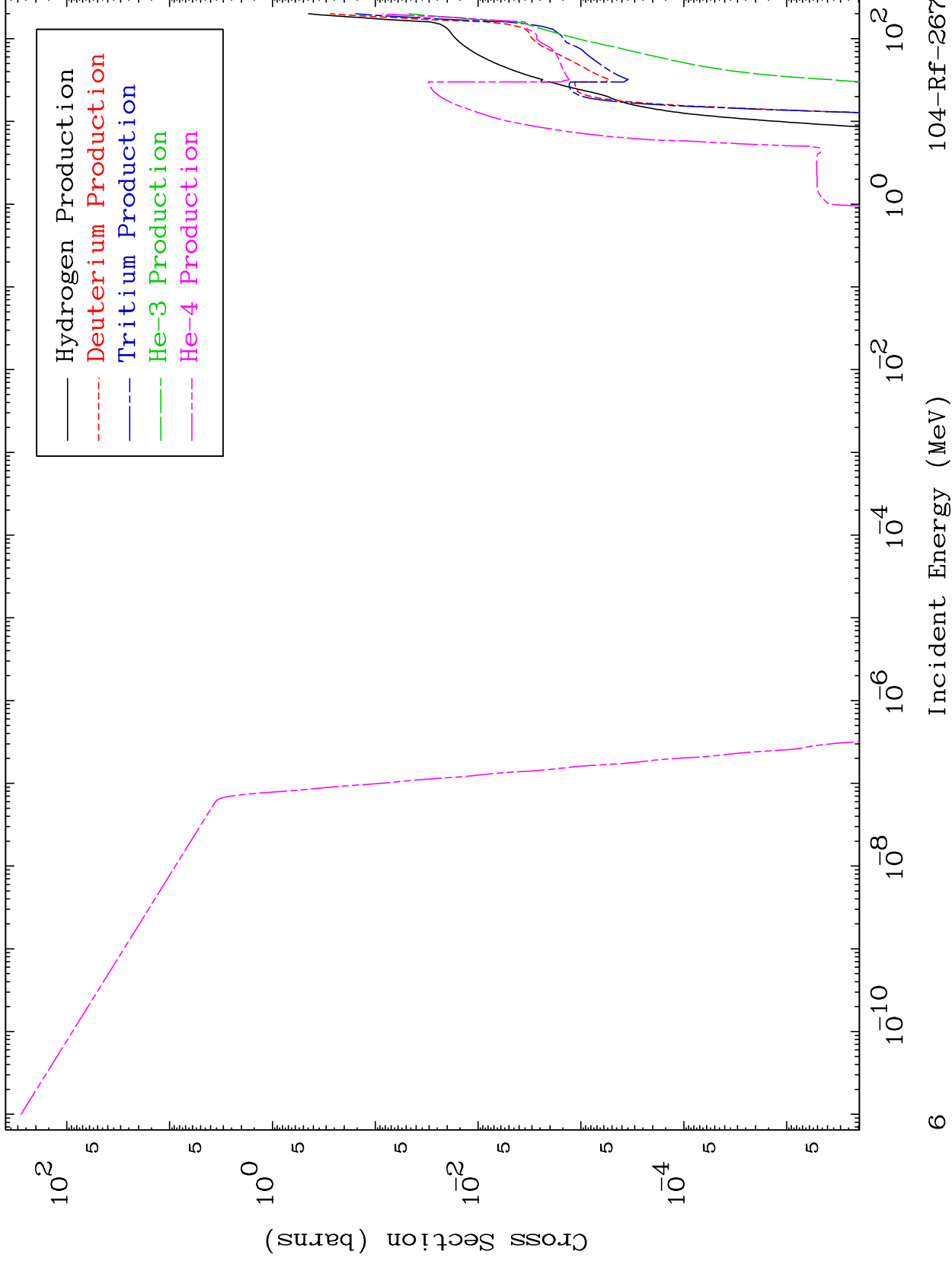
MAT 467

Charged Particle
293 Kelvin Cross Sections

104-Rf-267



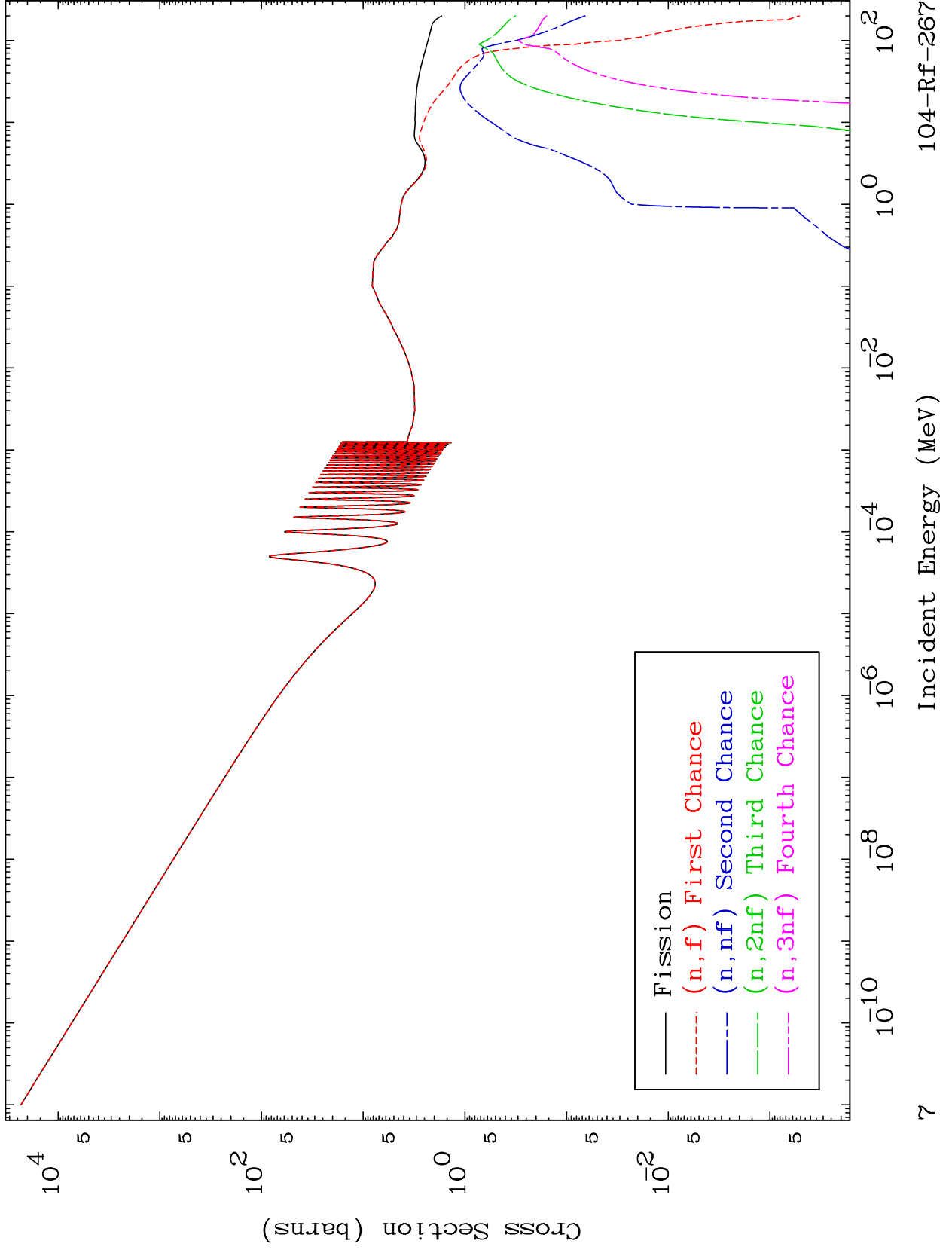


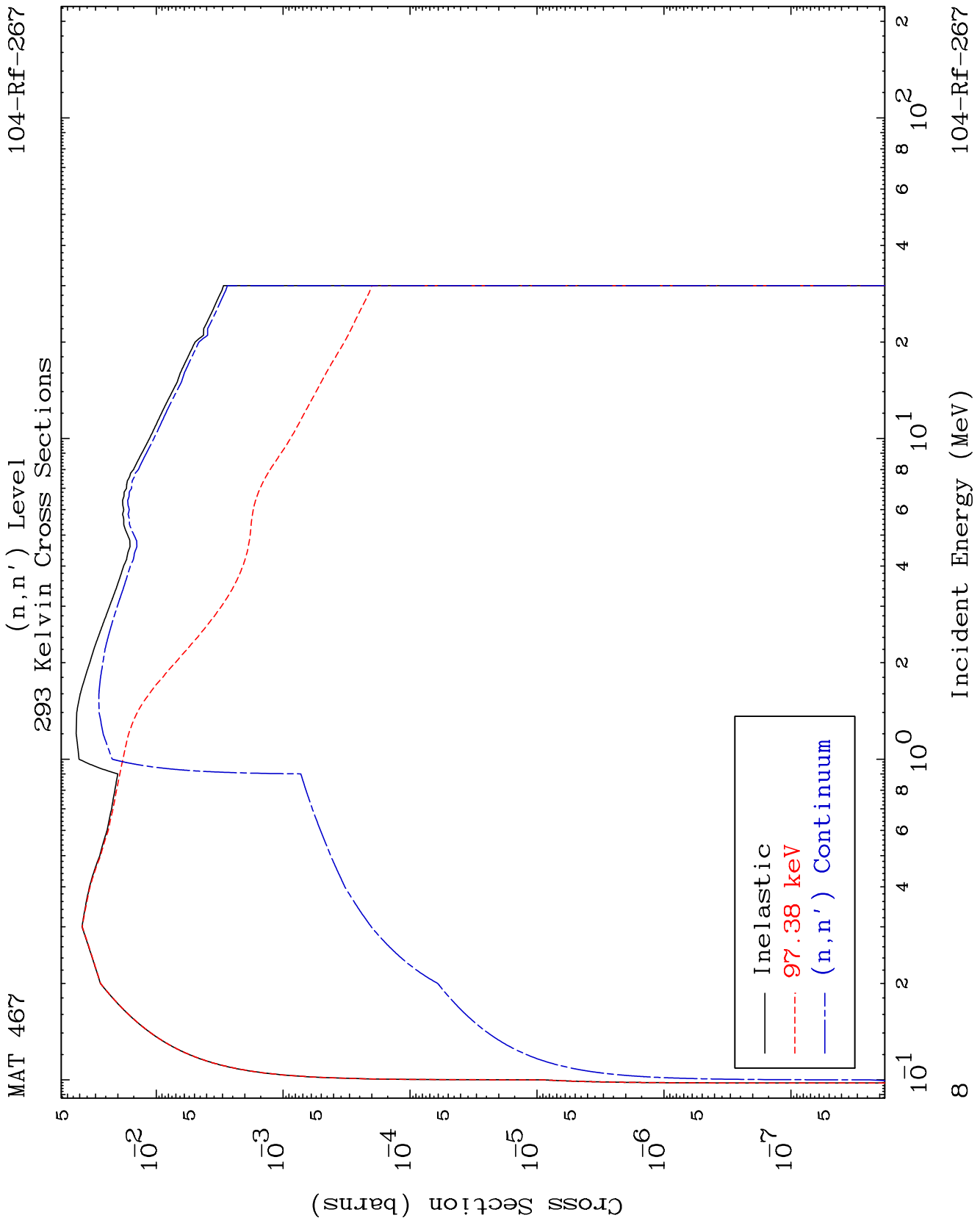


MAT 467

Fission
293 Kelvin Cross Sections

104-Rf-267

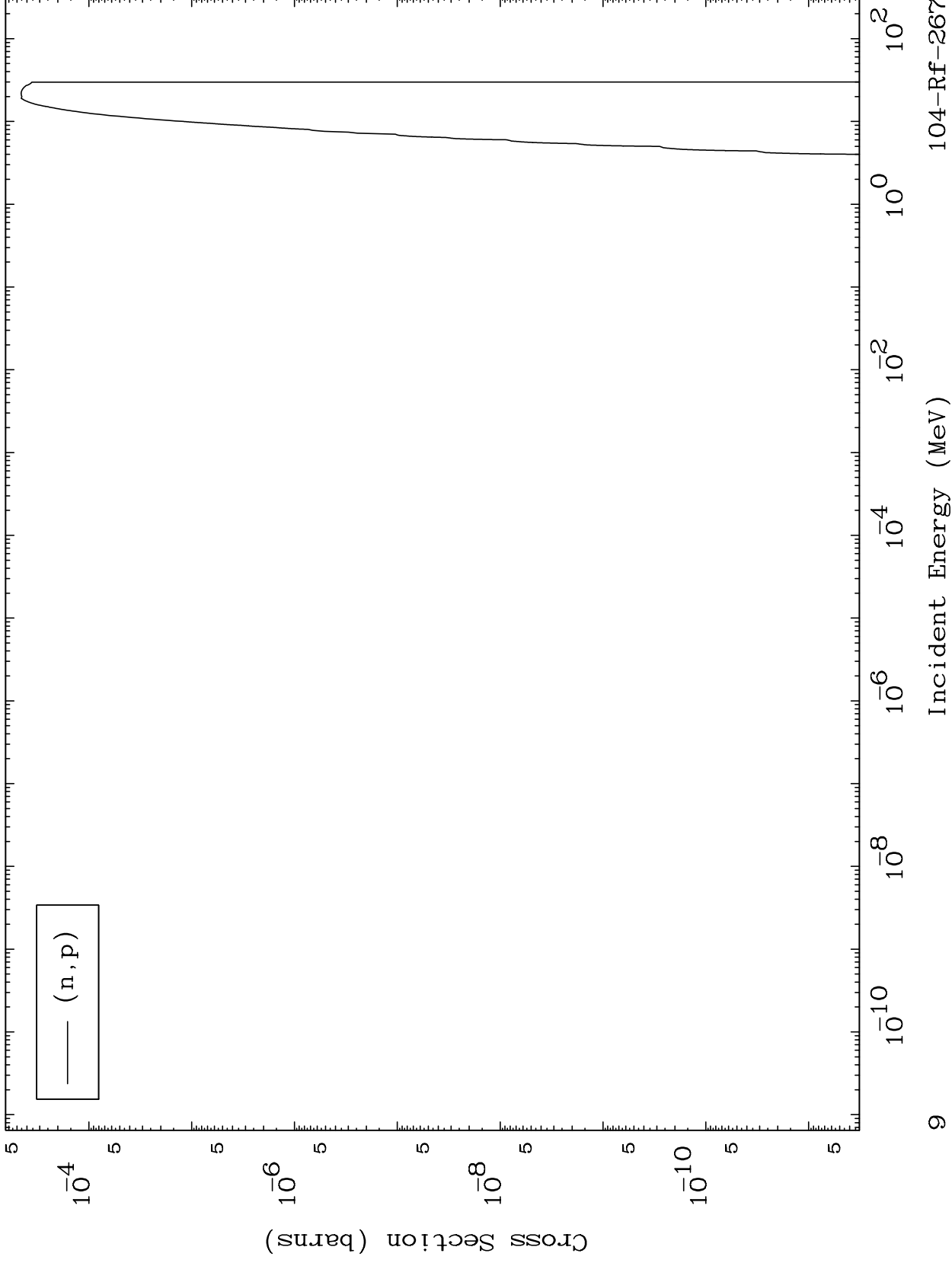




MAT 467

(n,p) Levels
293 Kelvin Cross Sections

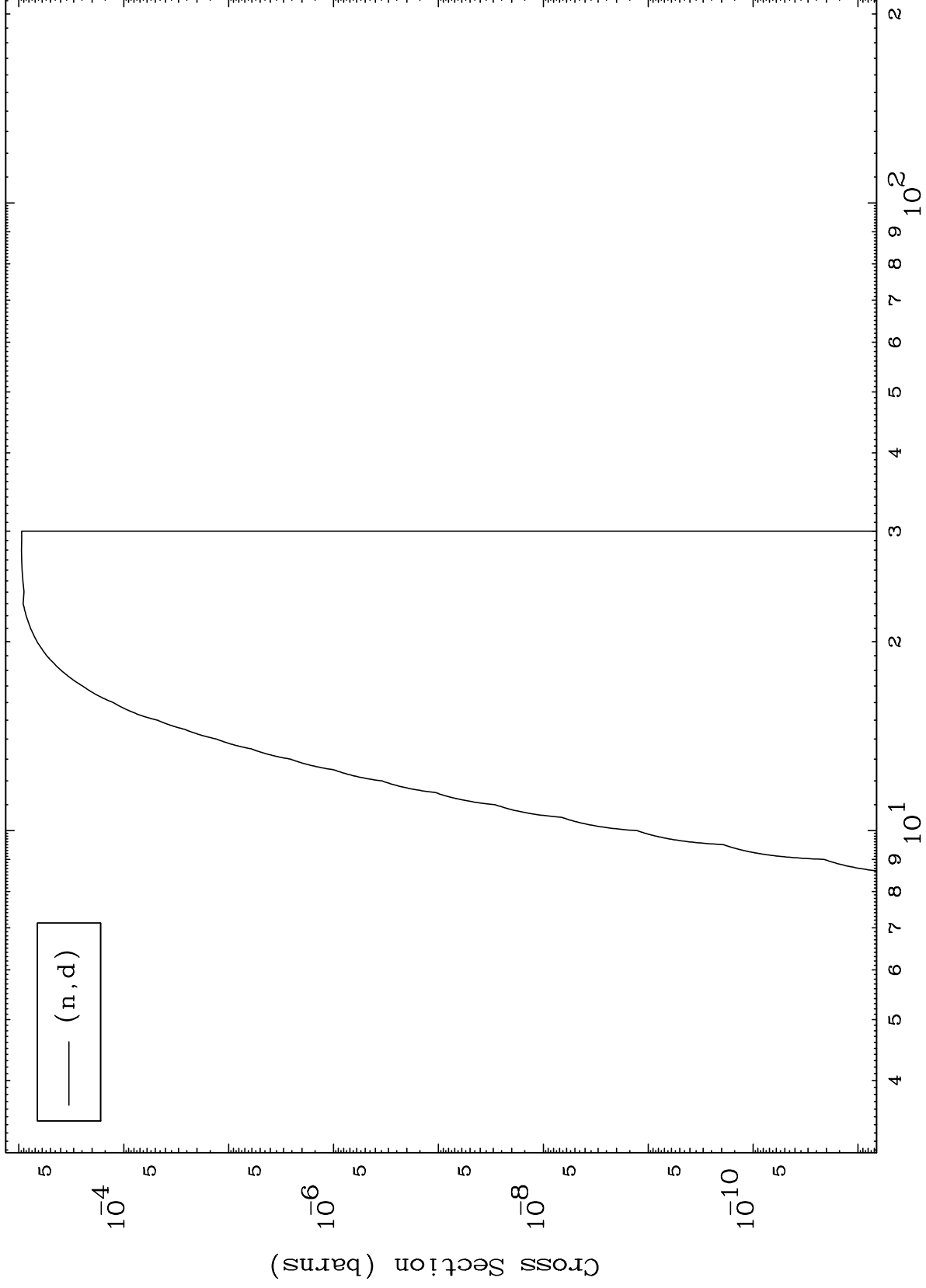
104-Rf-267



MAT 467

(n,d) Levels
293 Kelvin Cross Sections

104-Rf-267



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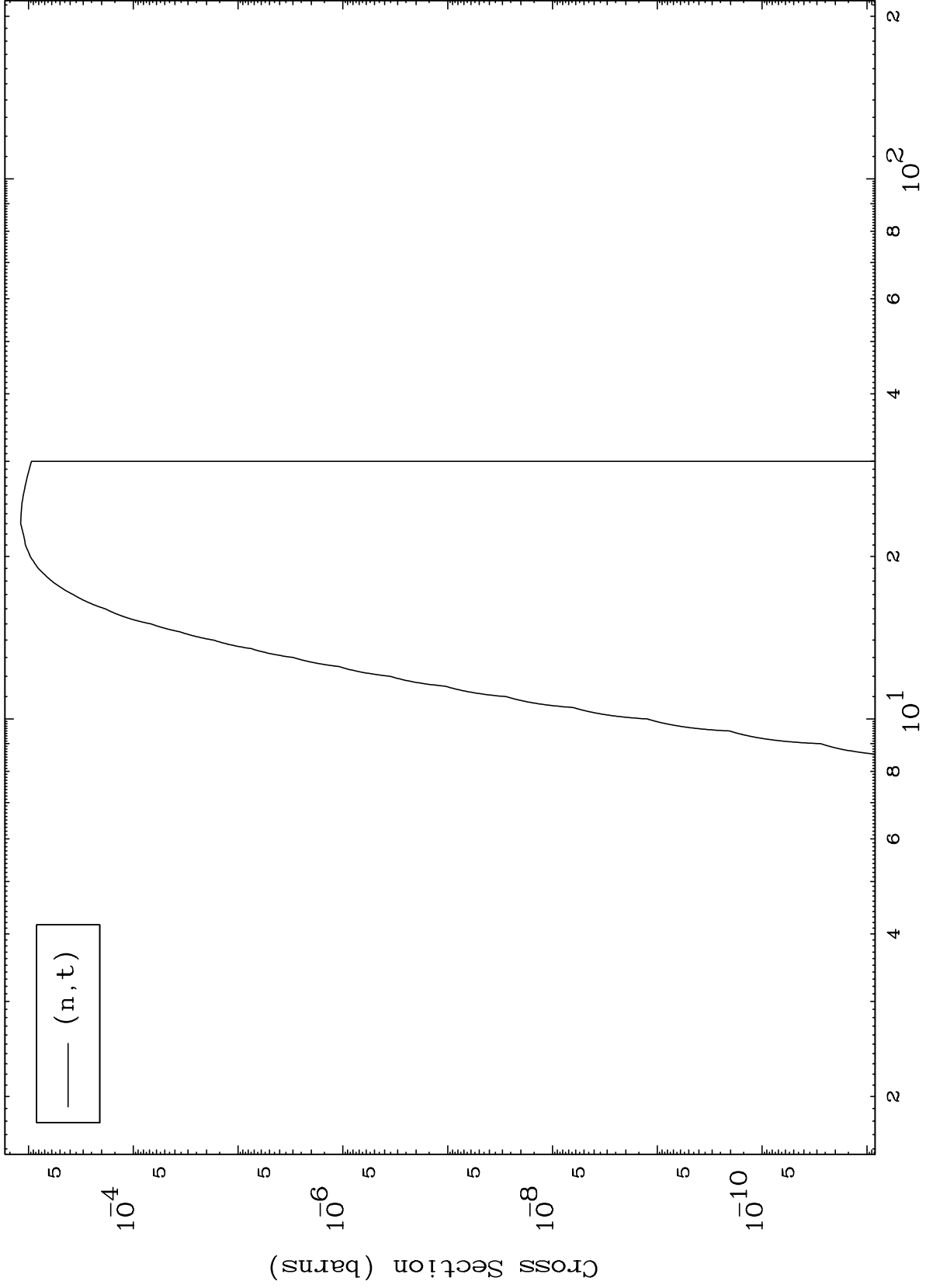
Incident Energy (MeV)

104-Rf-267

MAT 467

(n,t) Levels
293 Kelvin Cross Sections

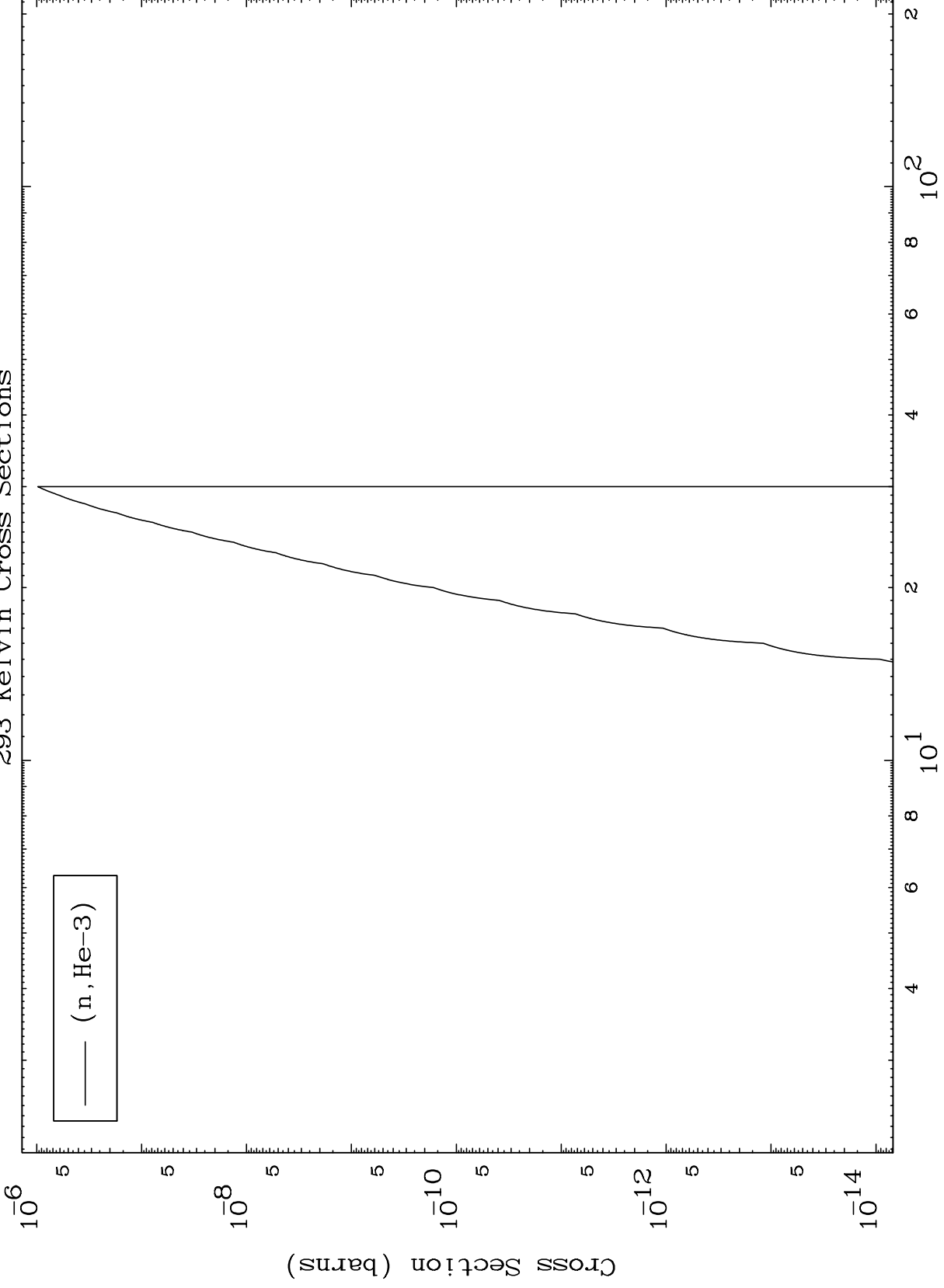
104-Rf-267



MAT 467

(n,He3) Levels
293 Kelvin Cross Sections

104-Rf-267



12

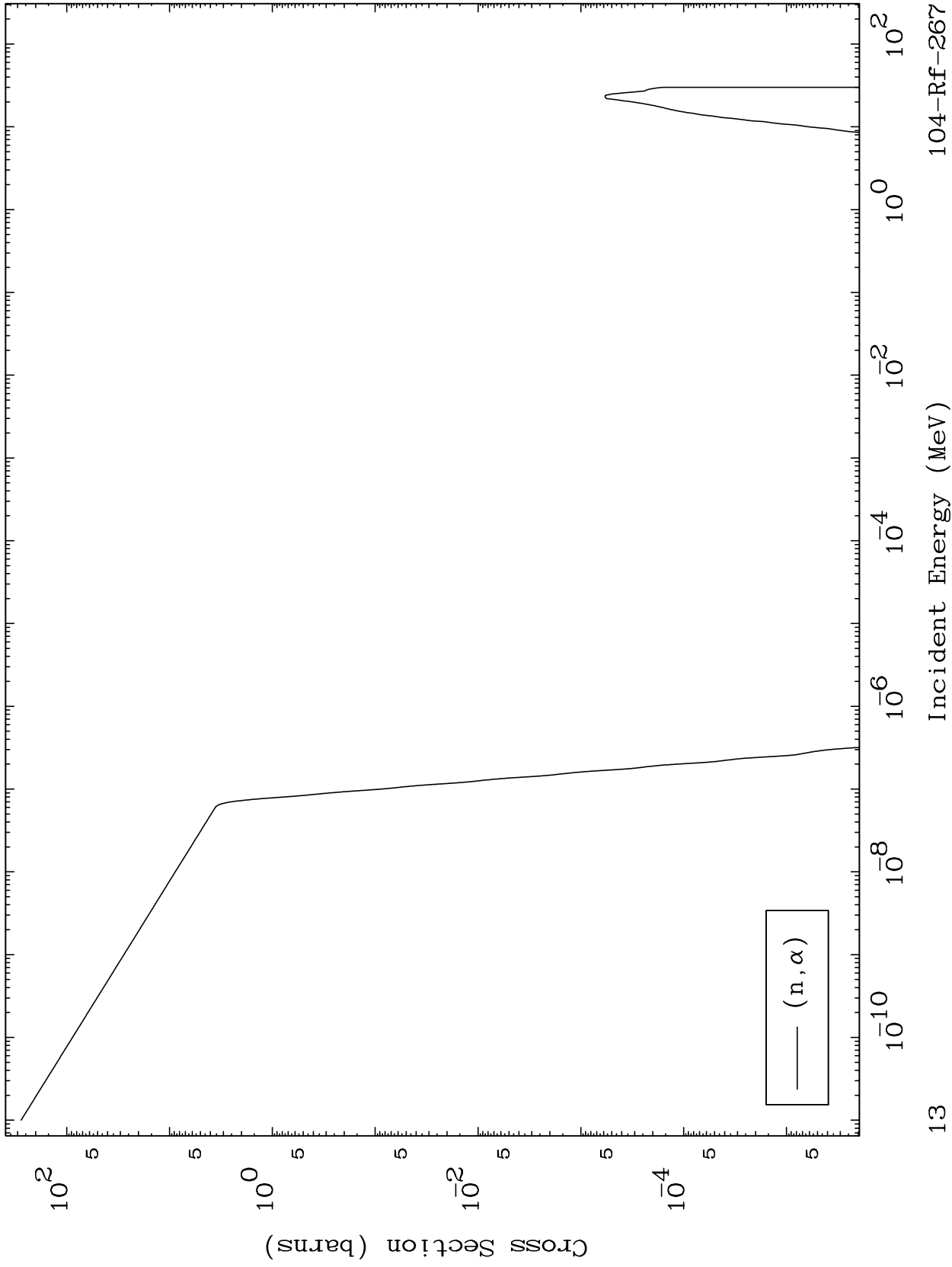
Incident Energy (MeV)

104-Rf-267

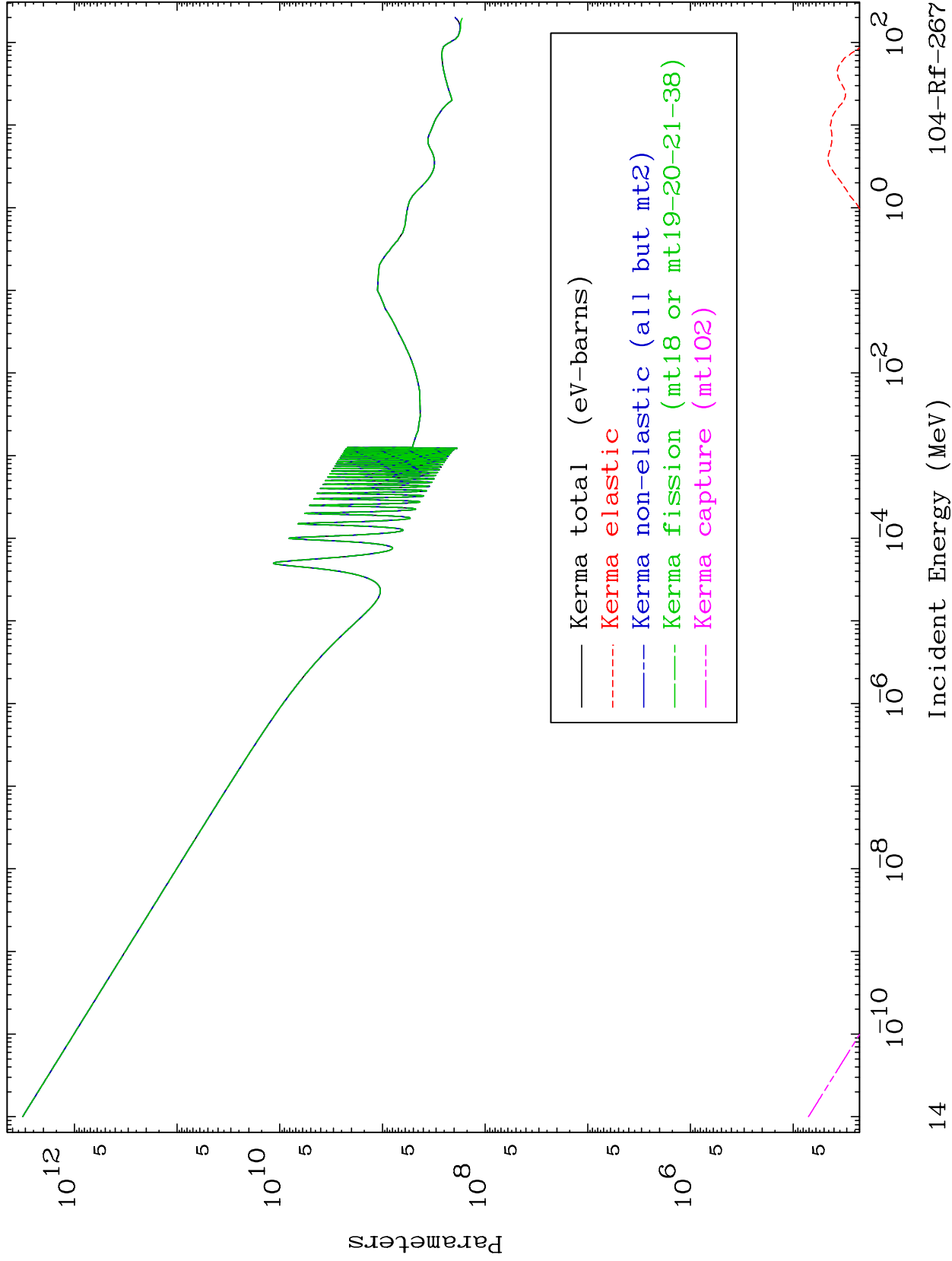
MAT 467

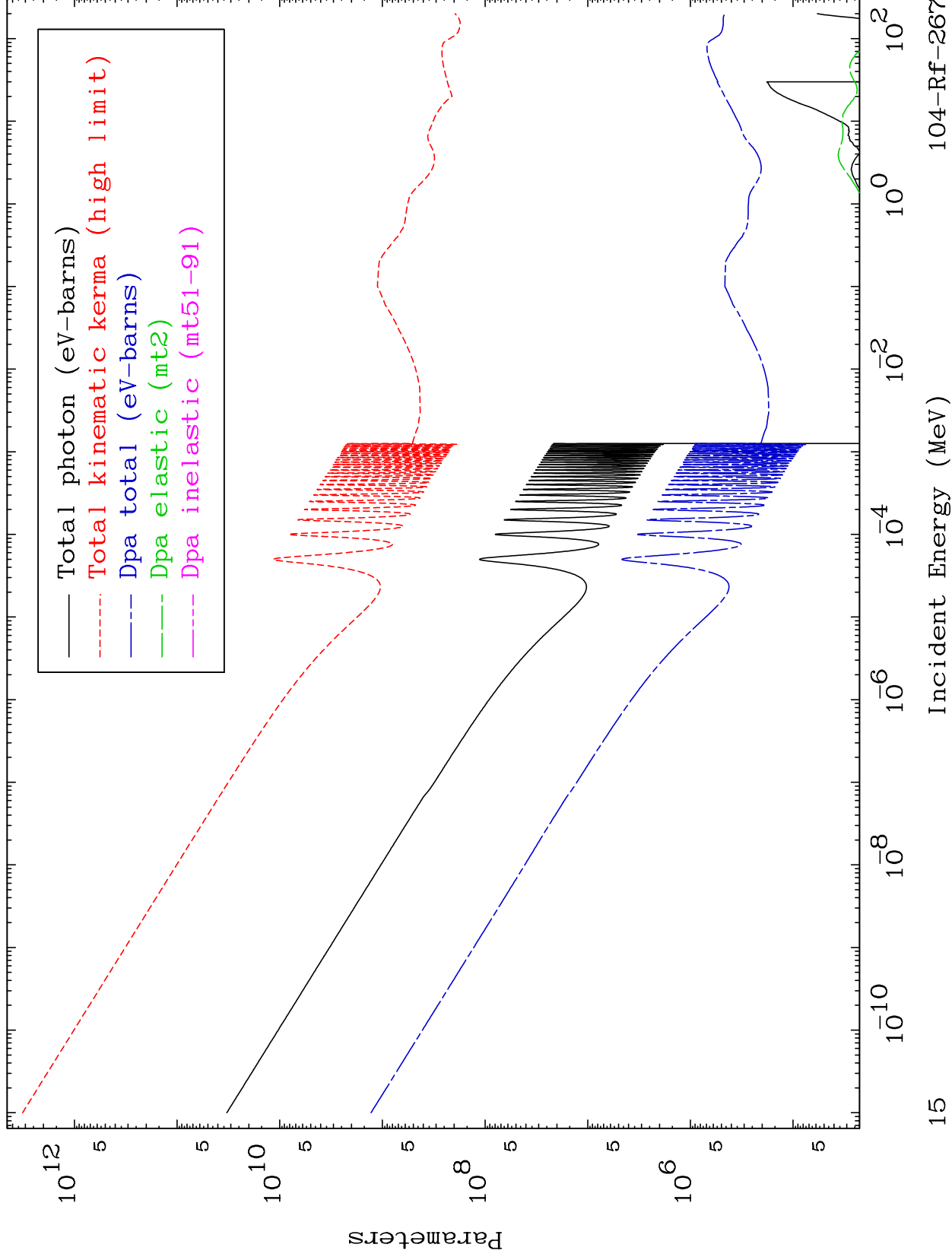
(n, α) Levels
293 Kelvin Cross Sections

104-Rf-267



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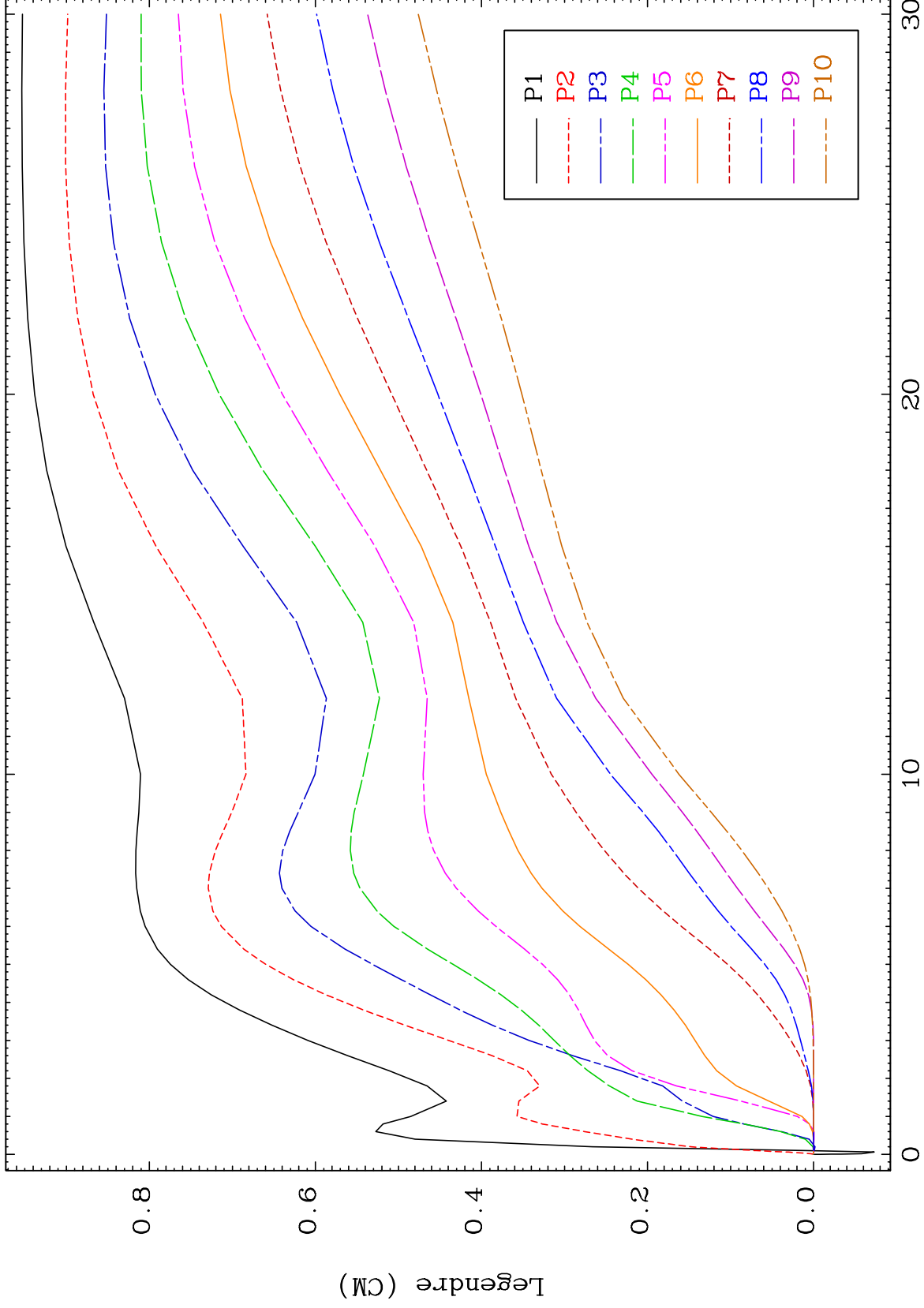




MAT 467

Elastic Legendre Coefficients

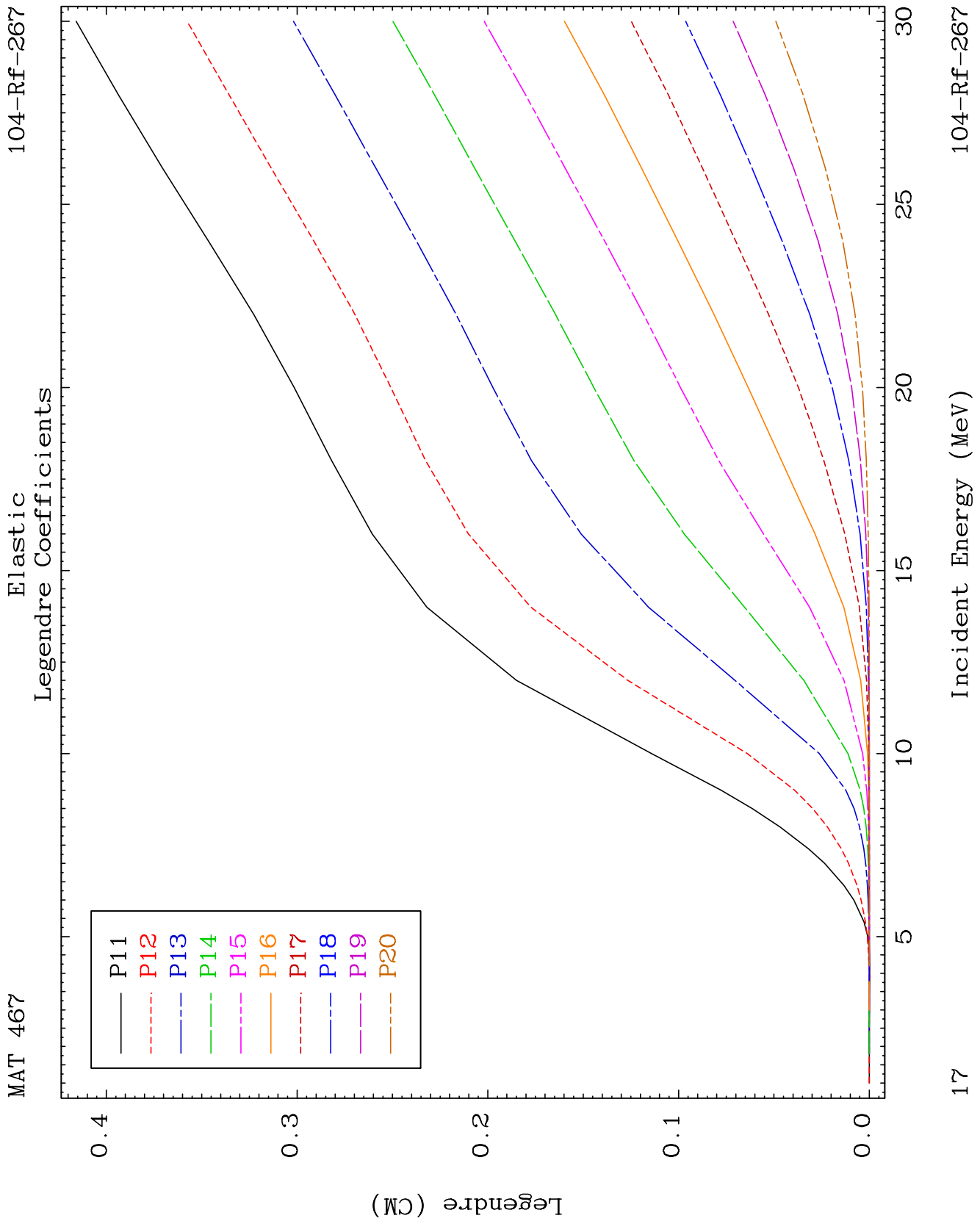
104-Rf-267



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Incident Energy (MeV)

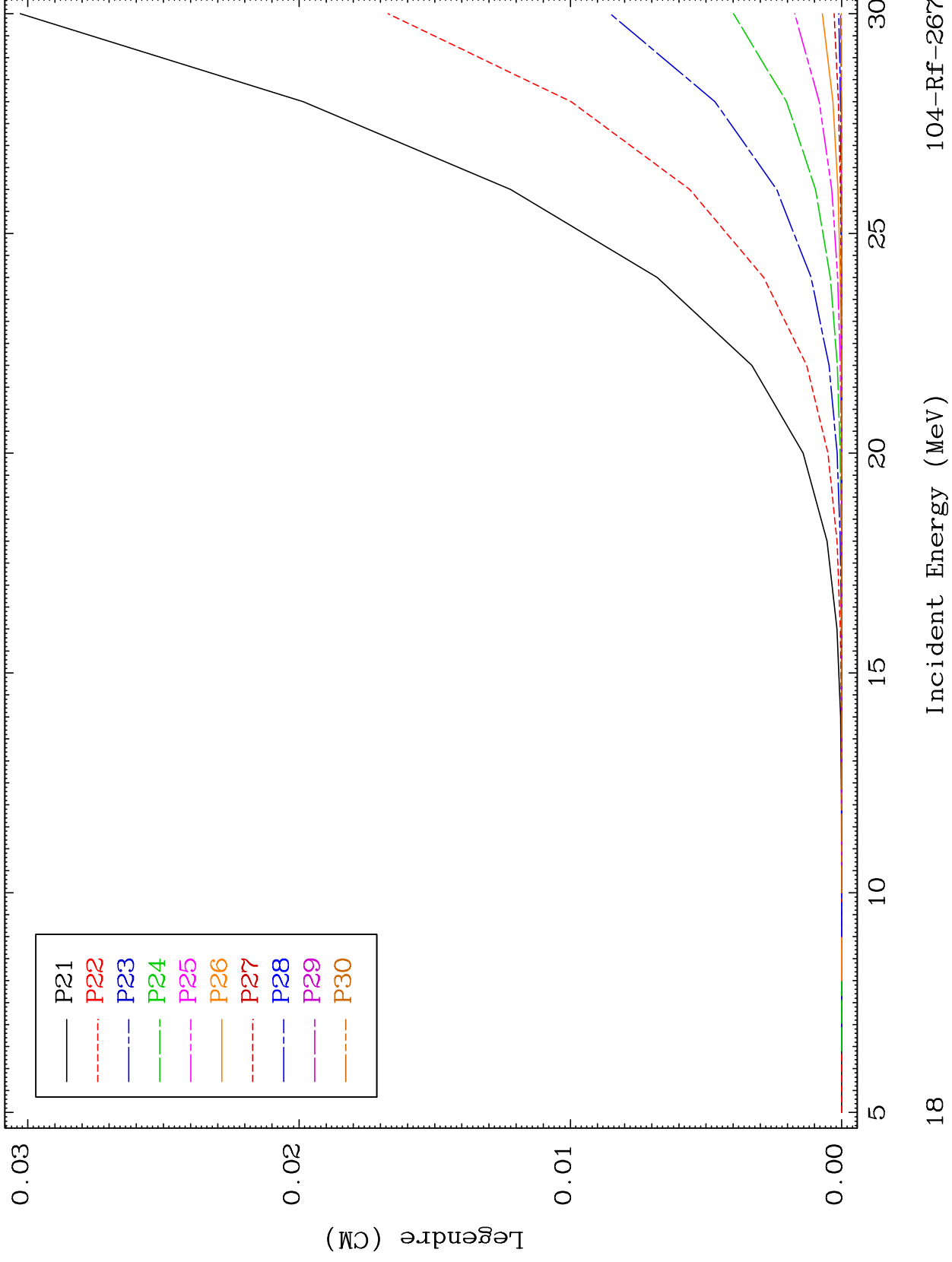
104-Rf-267



MAT 467

Elastic Legendre Coefficients

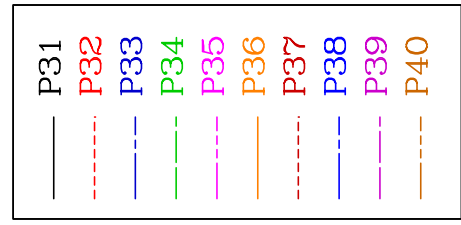
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MAT 467

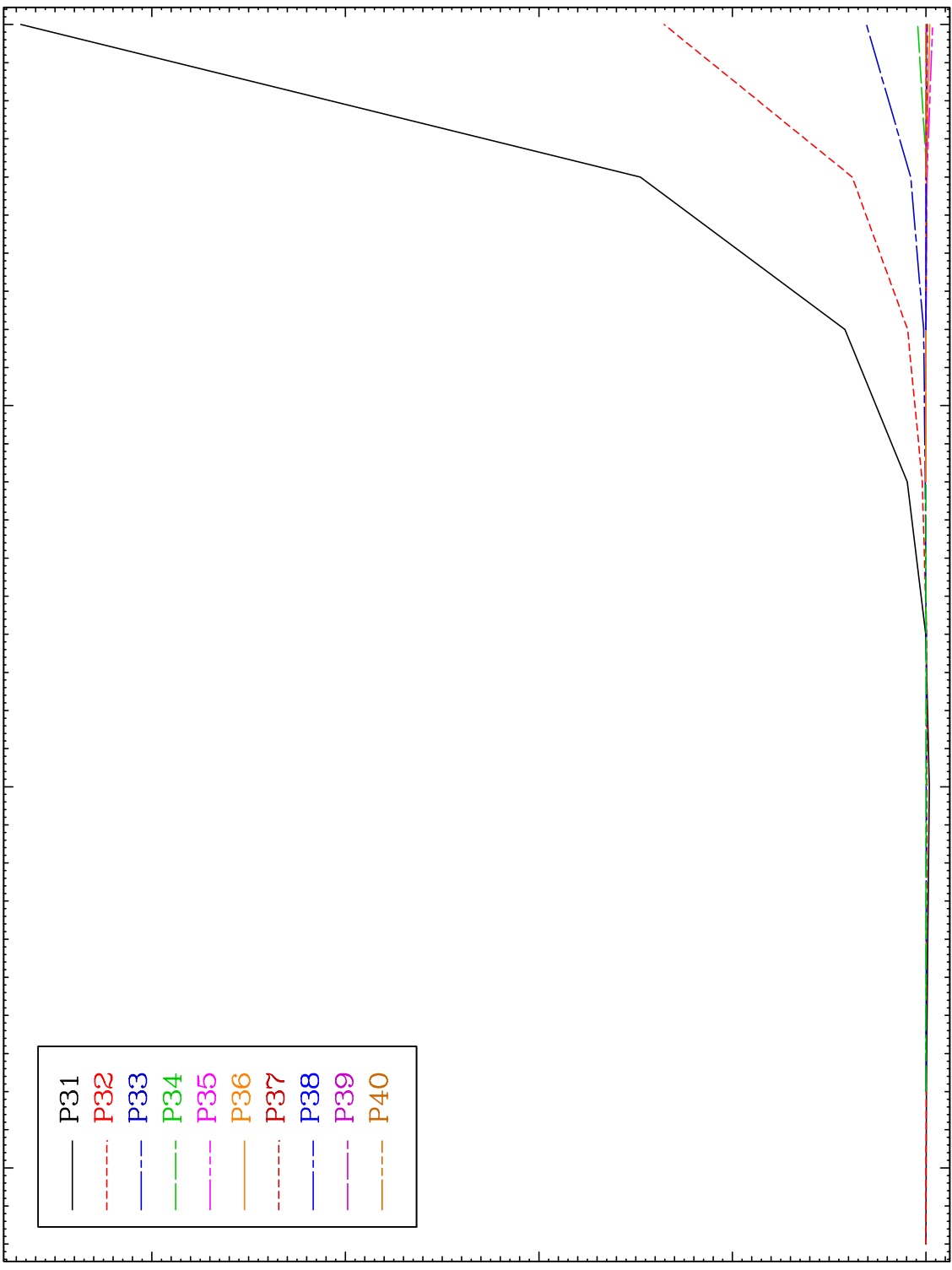
Elastic Legendre Coefficients

104-Rf-267



$\times 10^{-6}$

Legendre (CM)



30

25

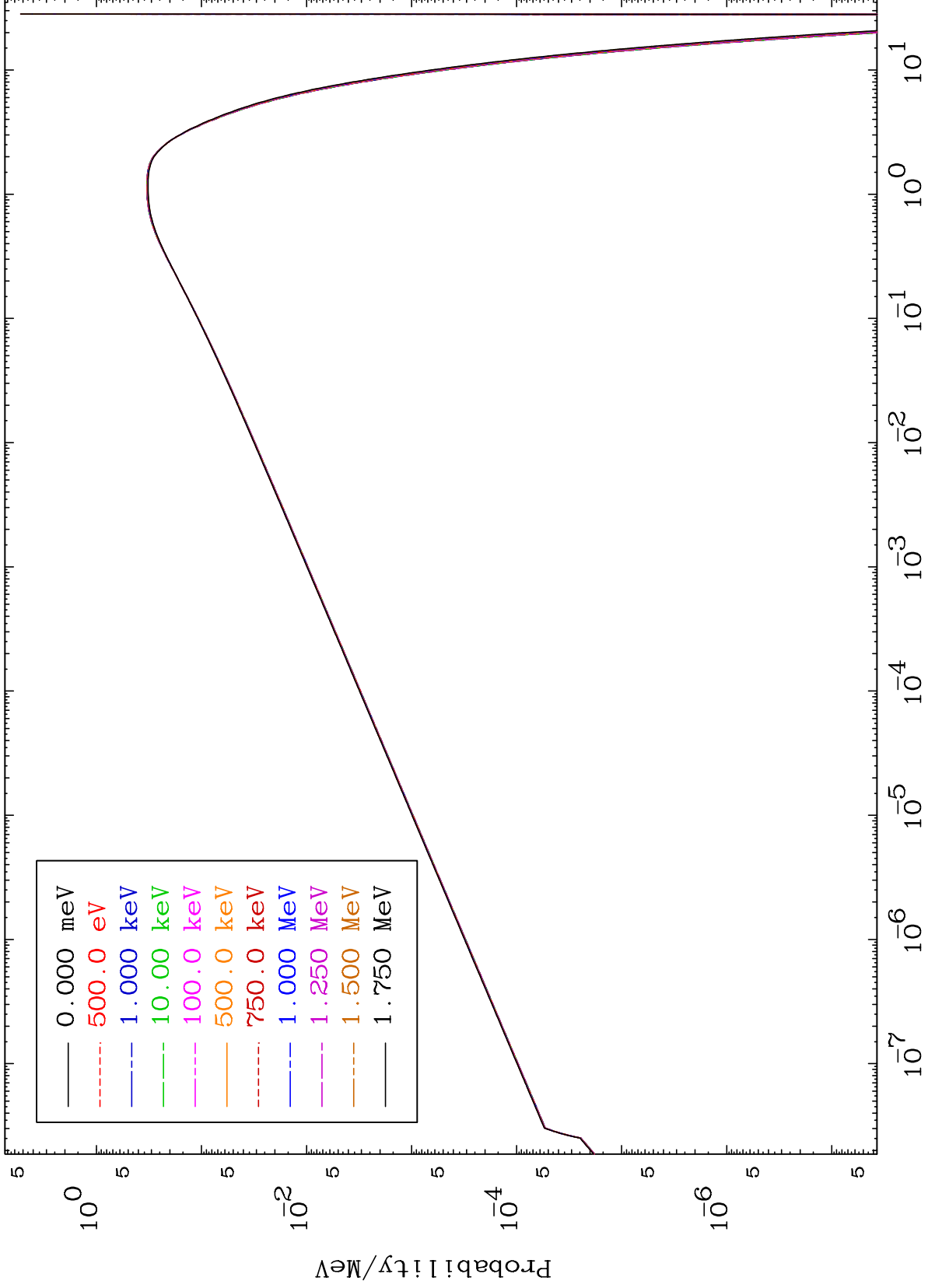
20

15

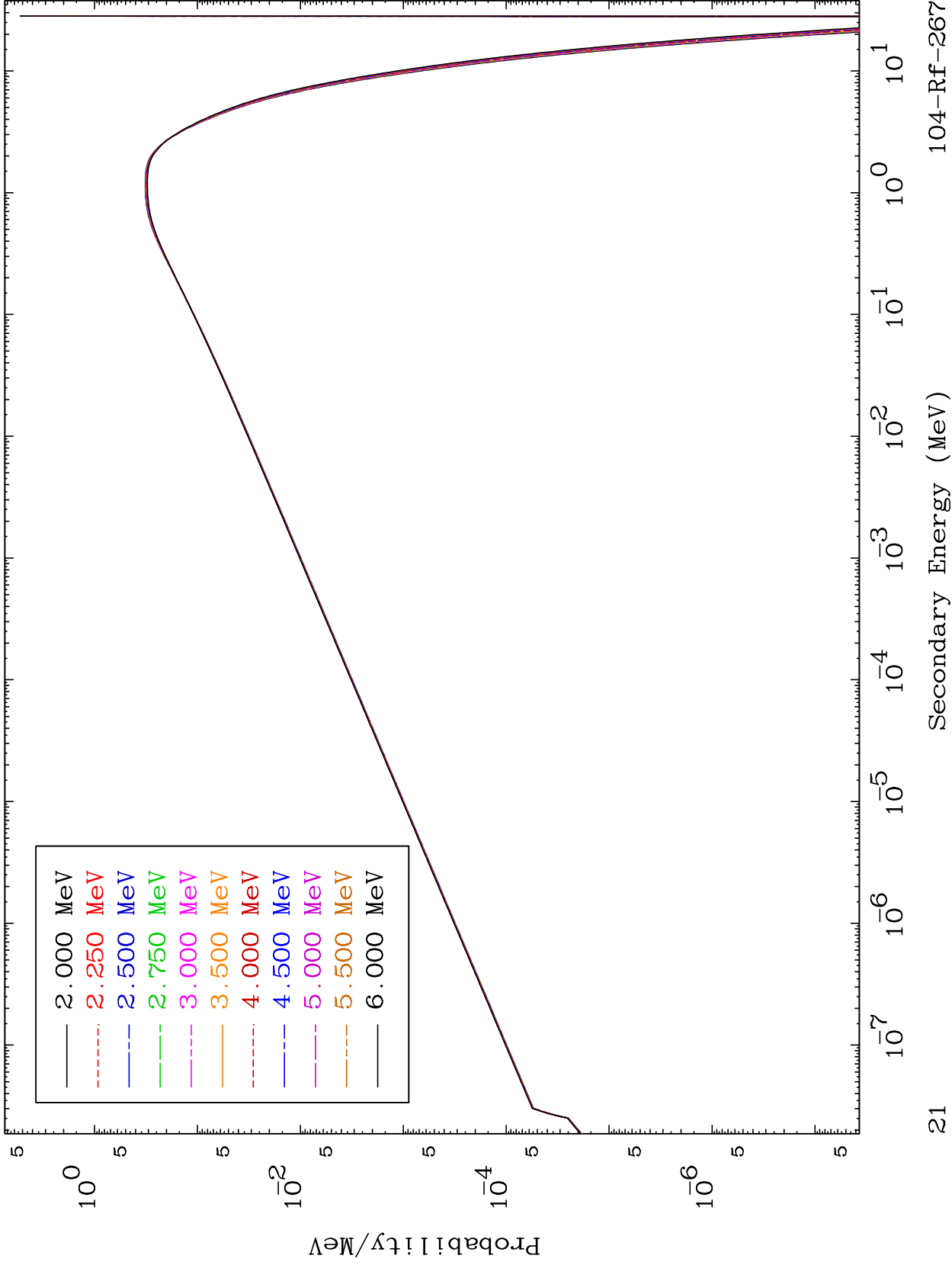
19

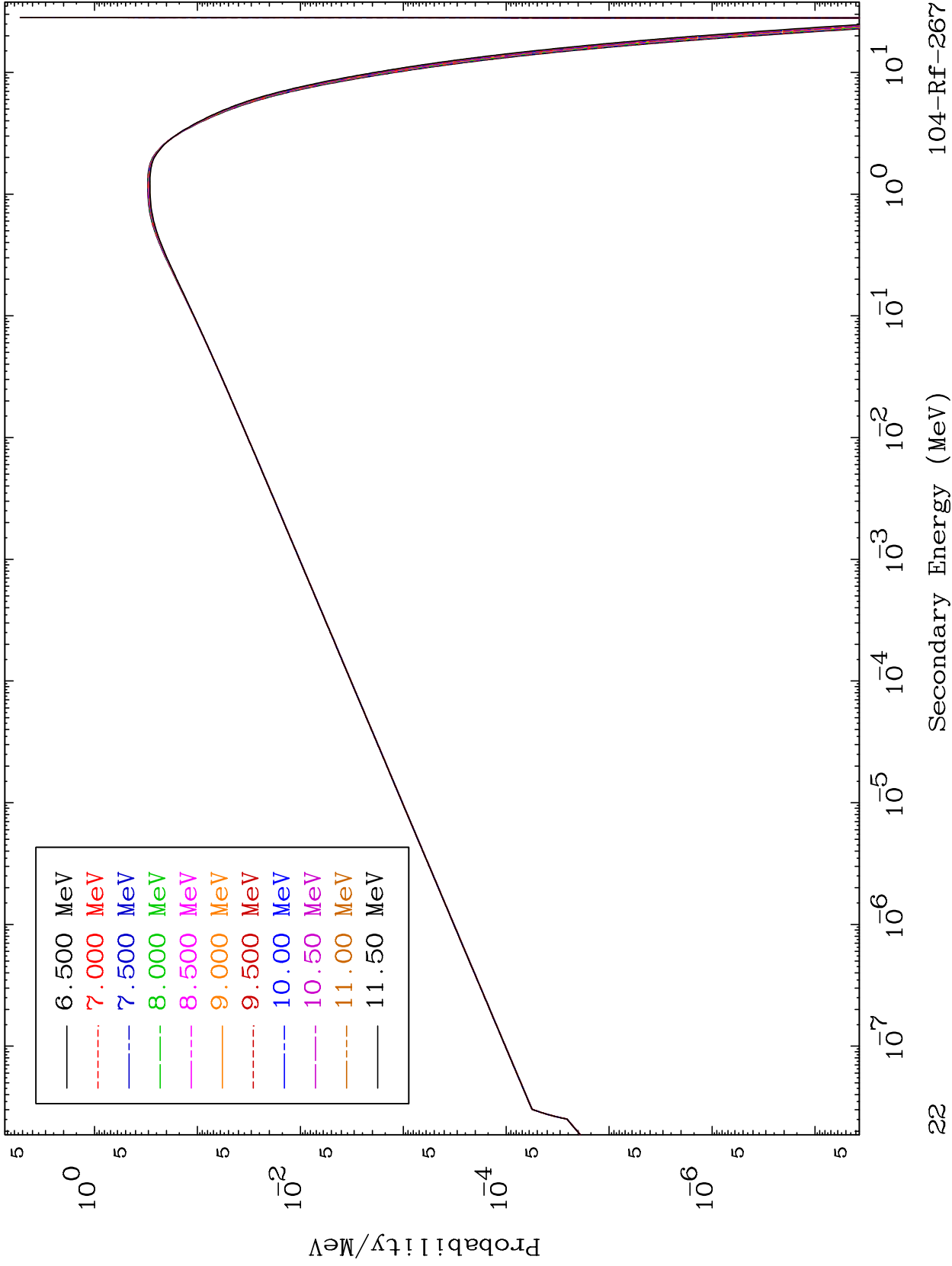
Incident Energy (MeV)

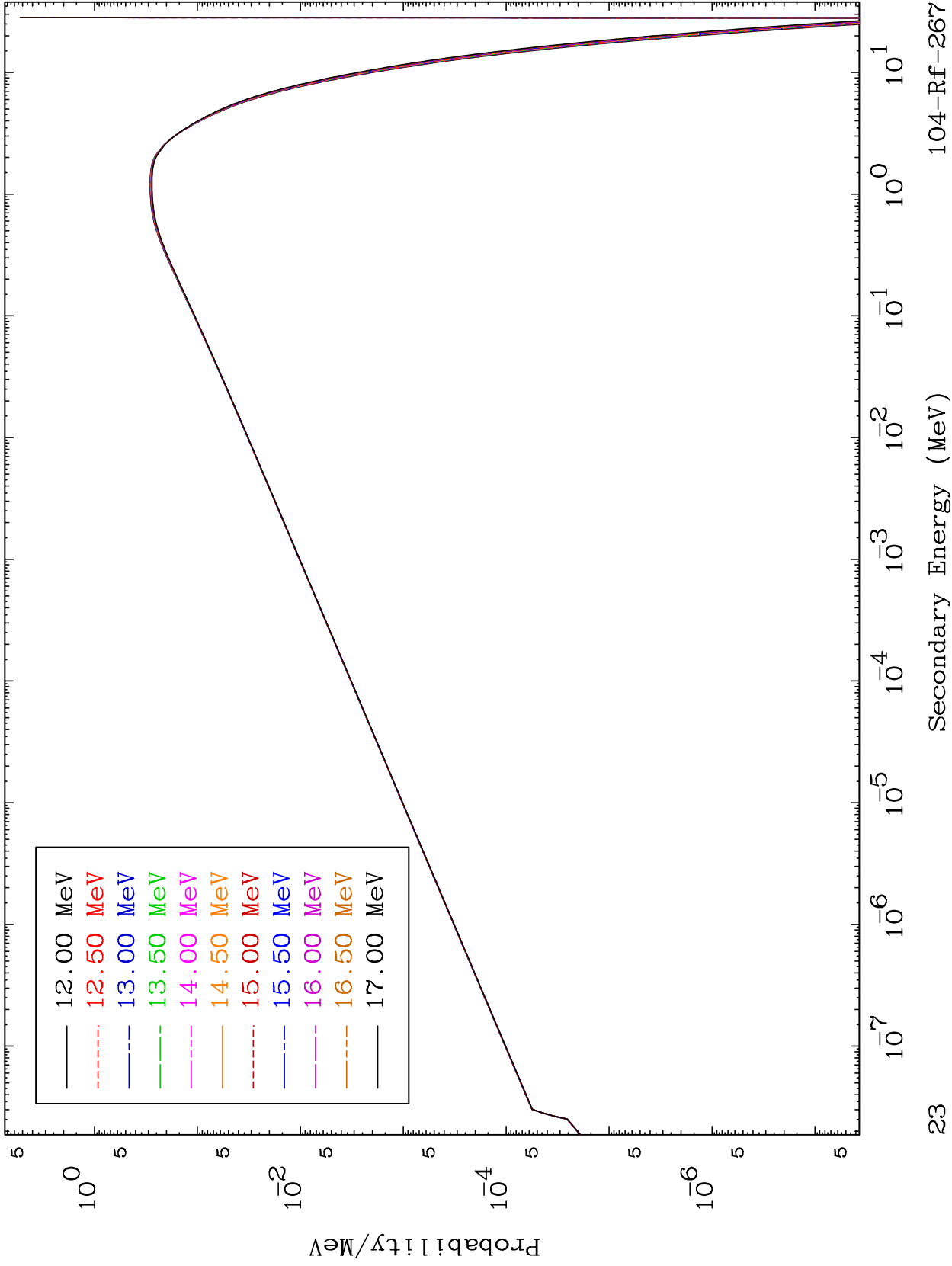
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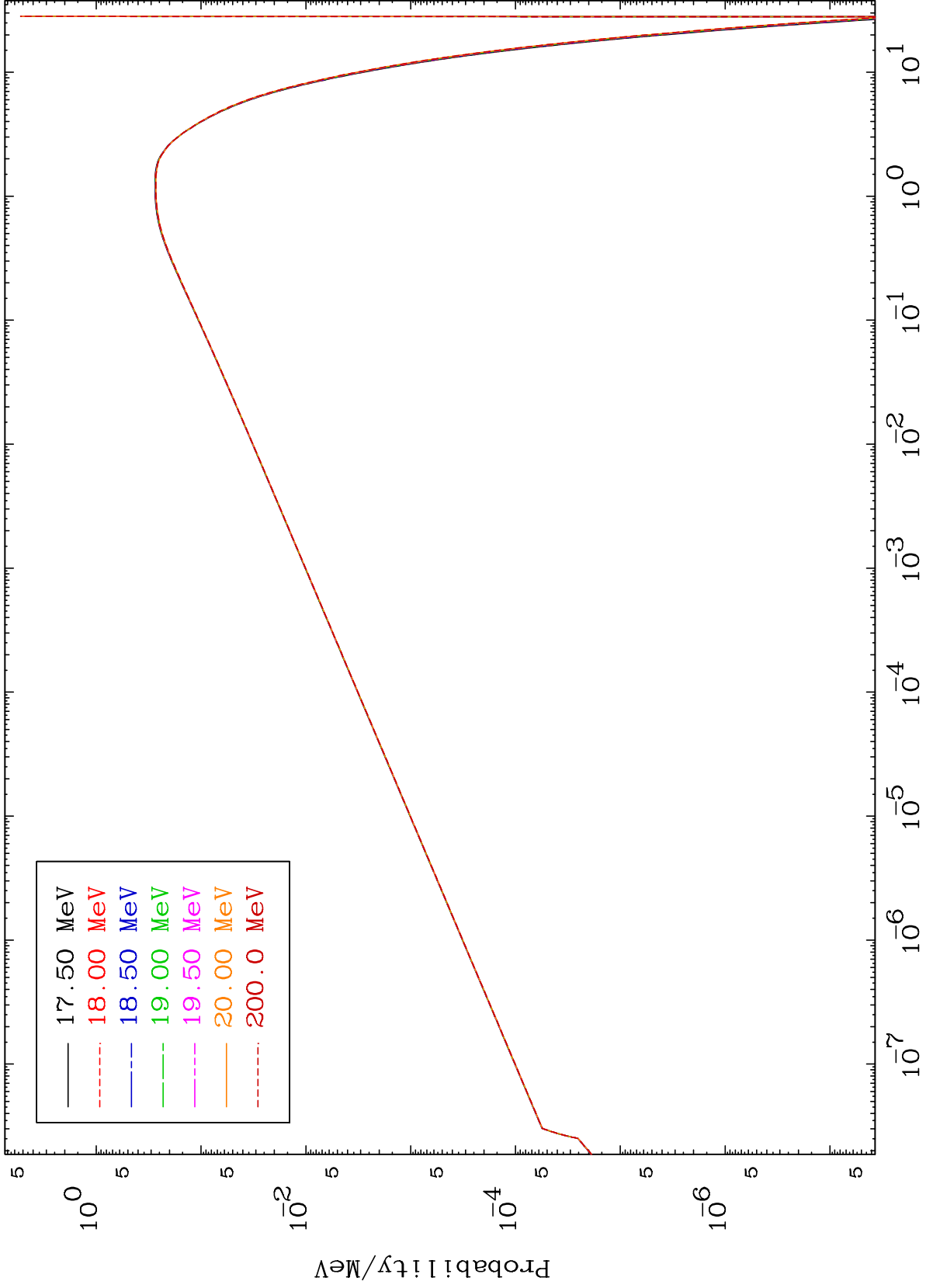
Fission
Energy Distribution







Fission
Energy Distribution

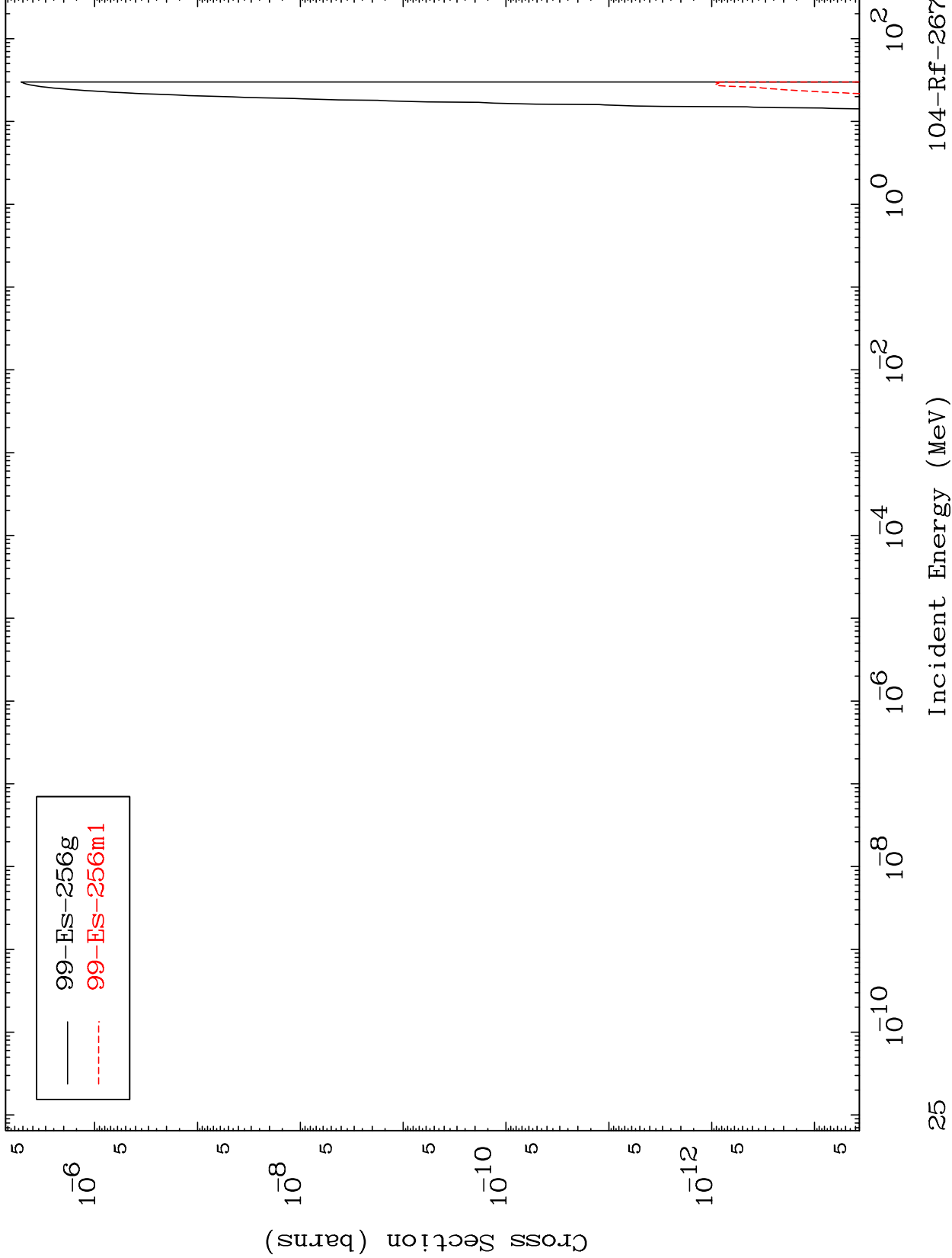


MAT 467

(n,n') t,2 α

104-Rf-267

Radionuclide Production Cross Section



25

Incident Energy (MeV)

104-Rf-267