

Program EVALPLOT
(Version 2015-2)

by

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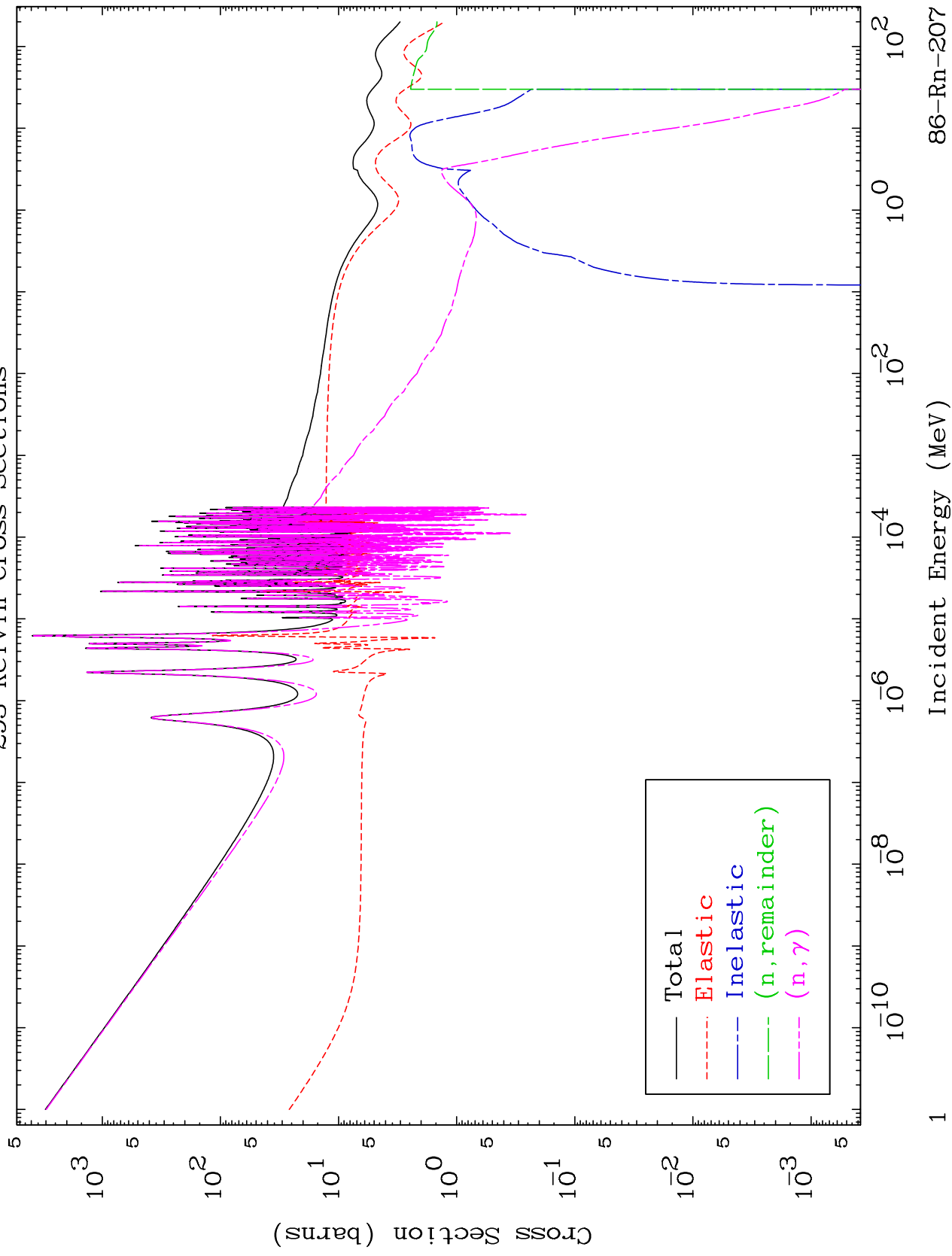
Press Mouse Button to Start

MAT 8613

Major

293 Kelvin Cross Sections

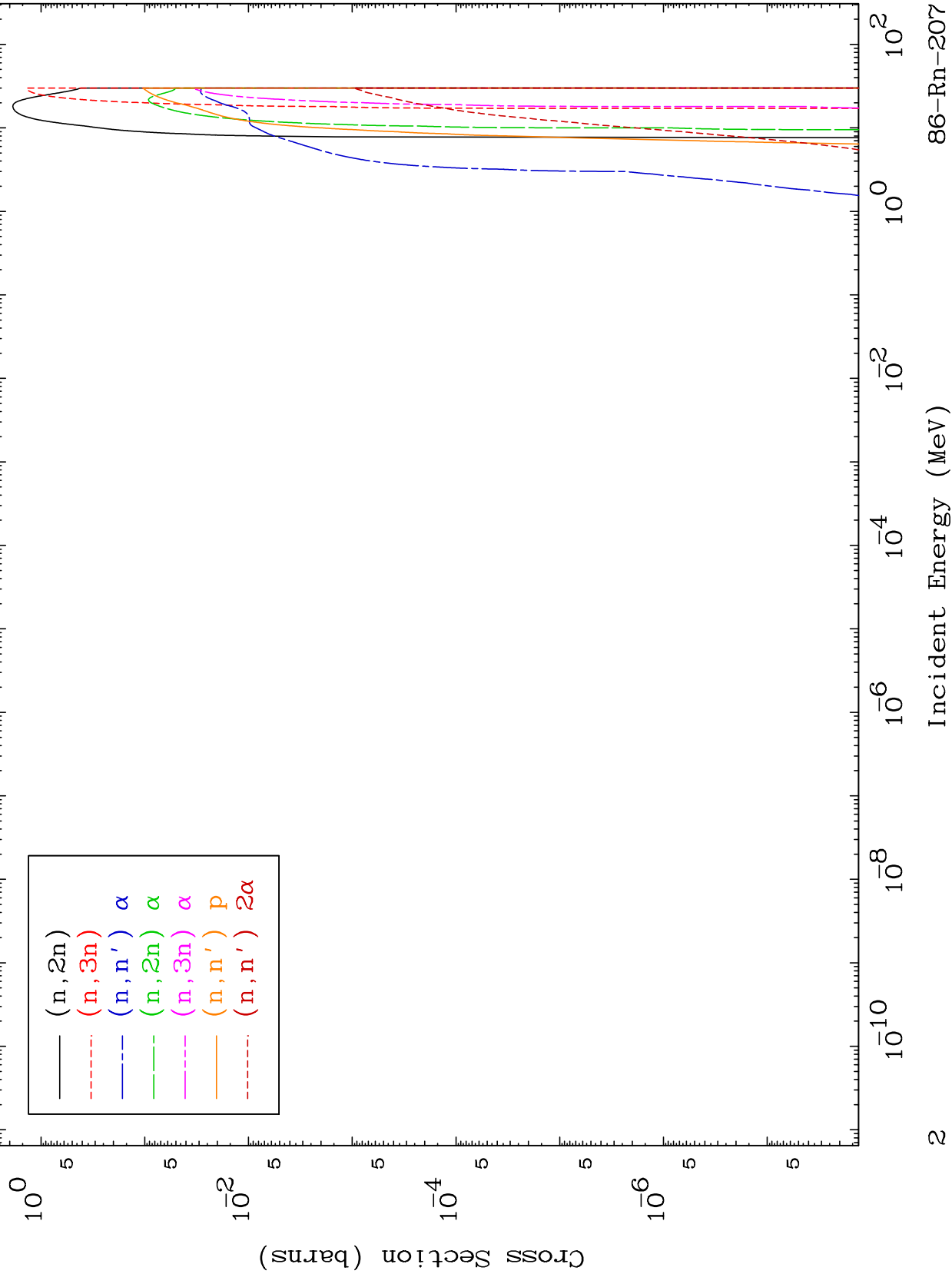
86-Rn-207



MAT 8613

Neutron Production
293 Kelvin Cross Sections

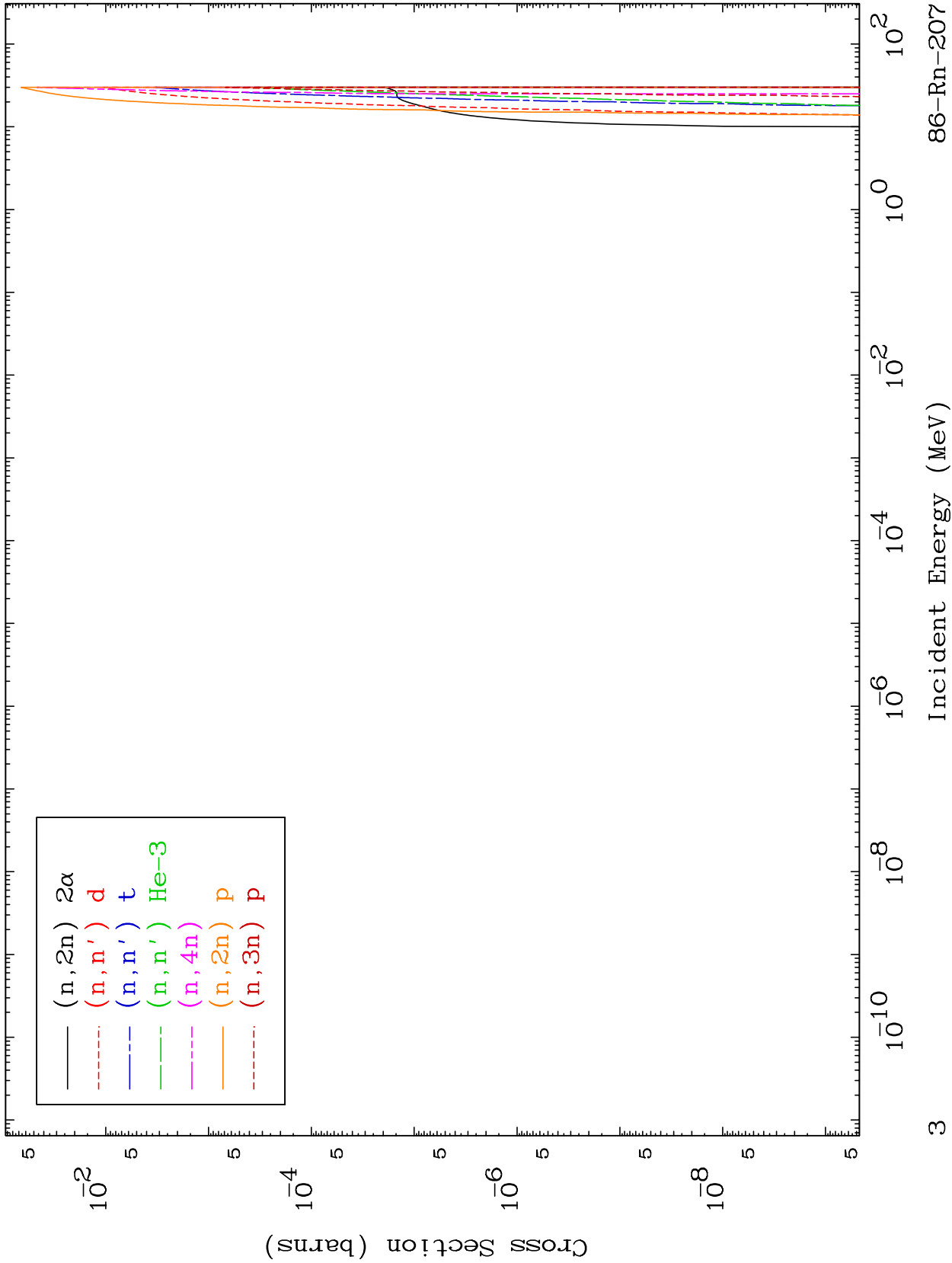
86-Rn-207



MAT 8613

Neutron Production
293 Kelvin Cross Sections

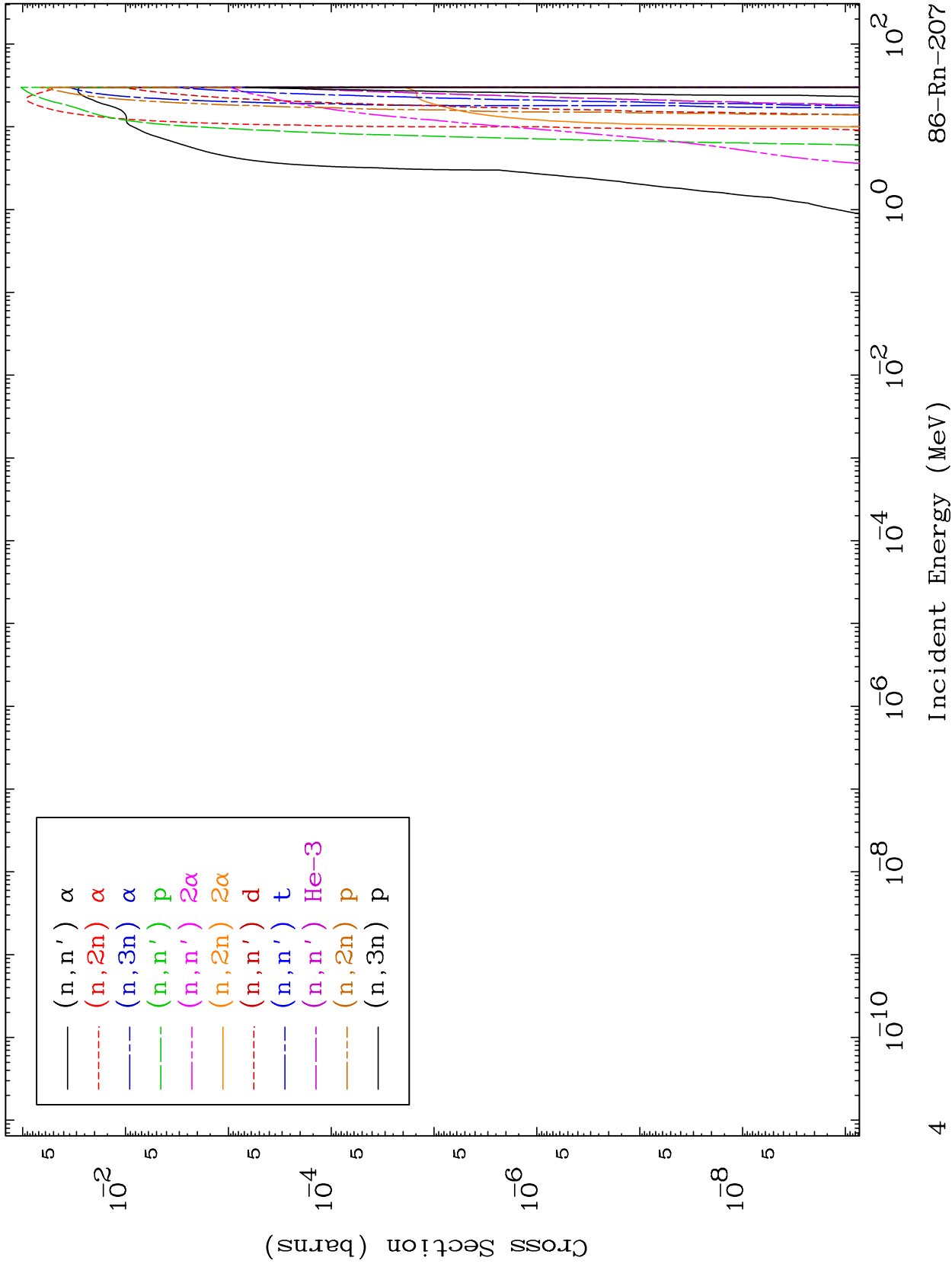
86-Rn-207



MAT 8613

Charged Particle
293 Kelvin Cross Sections

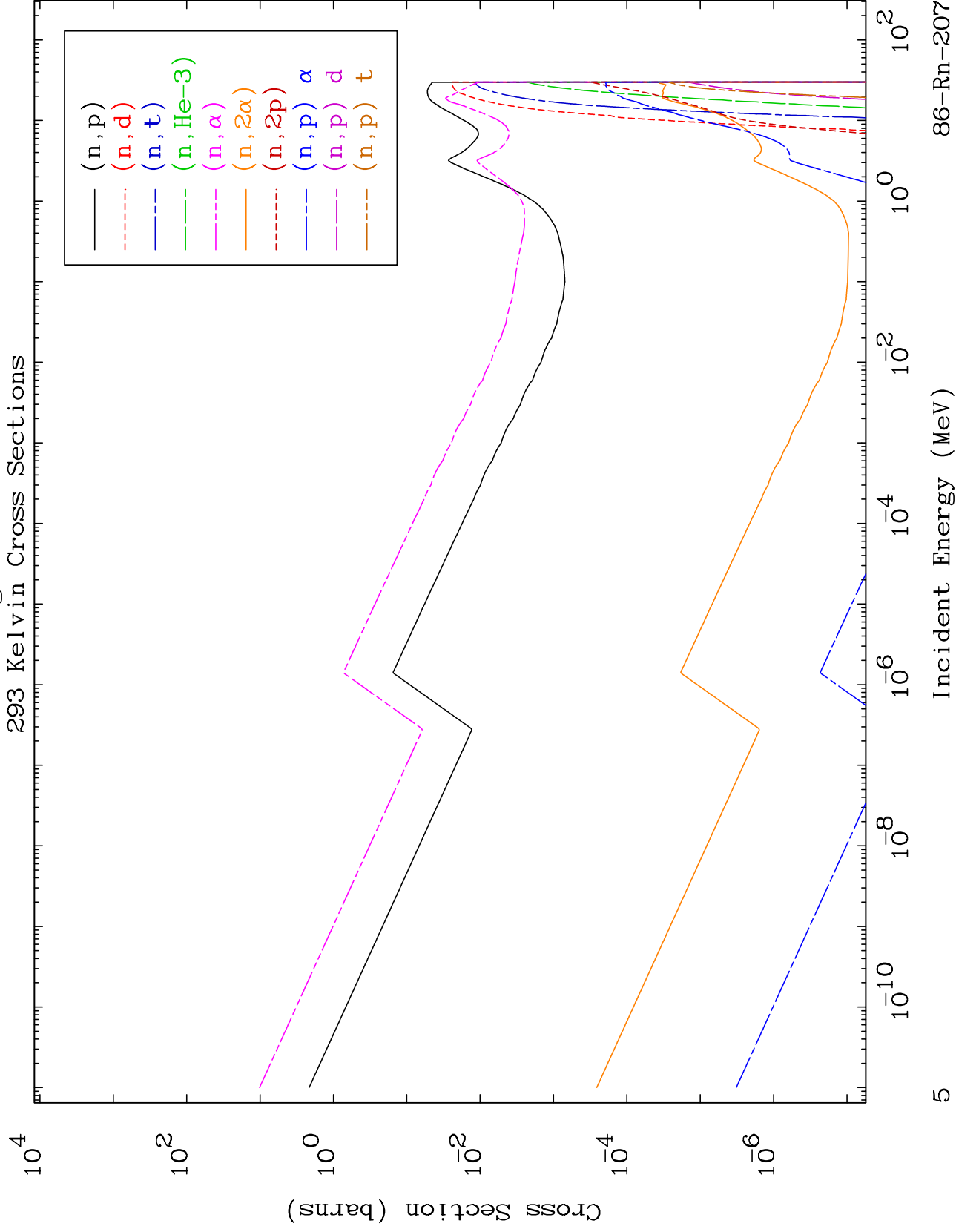
86-Rn-207



MAT 8613

Charged Particle
293 Kelvin Cross Sections

86-Rn-207



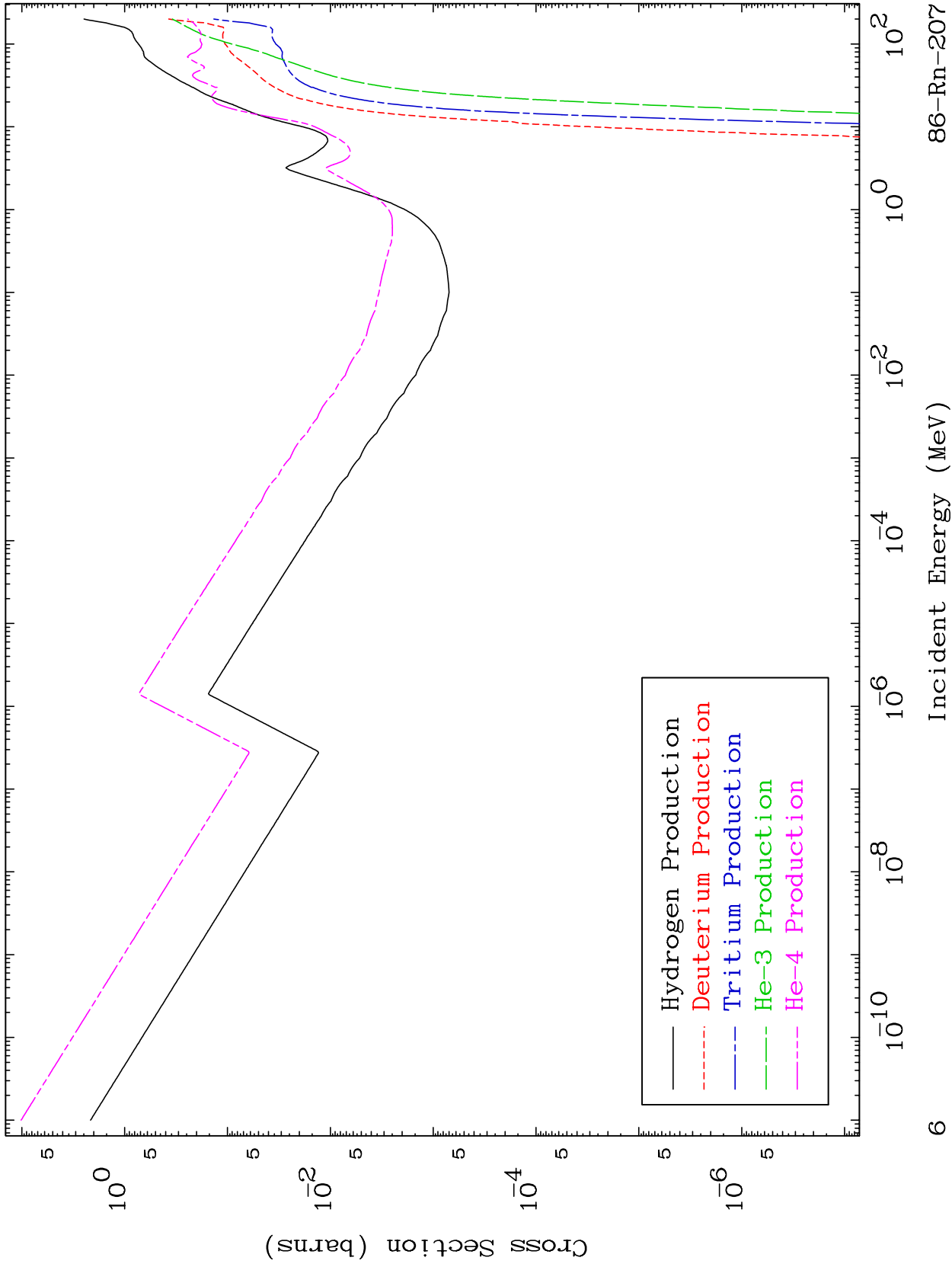
5

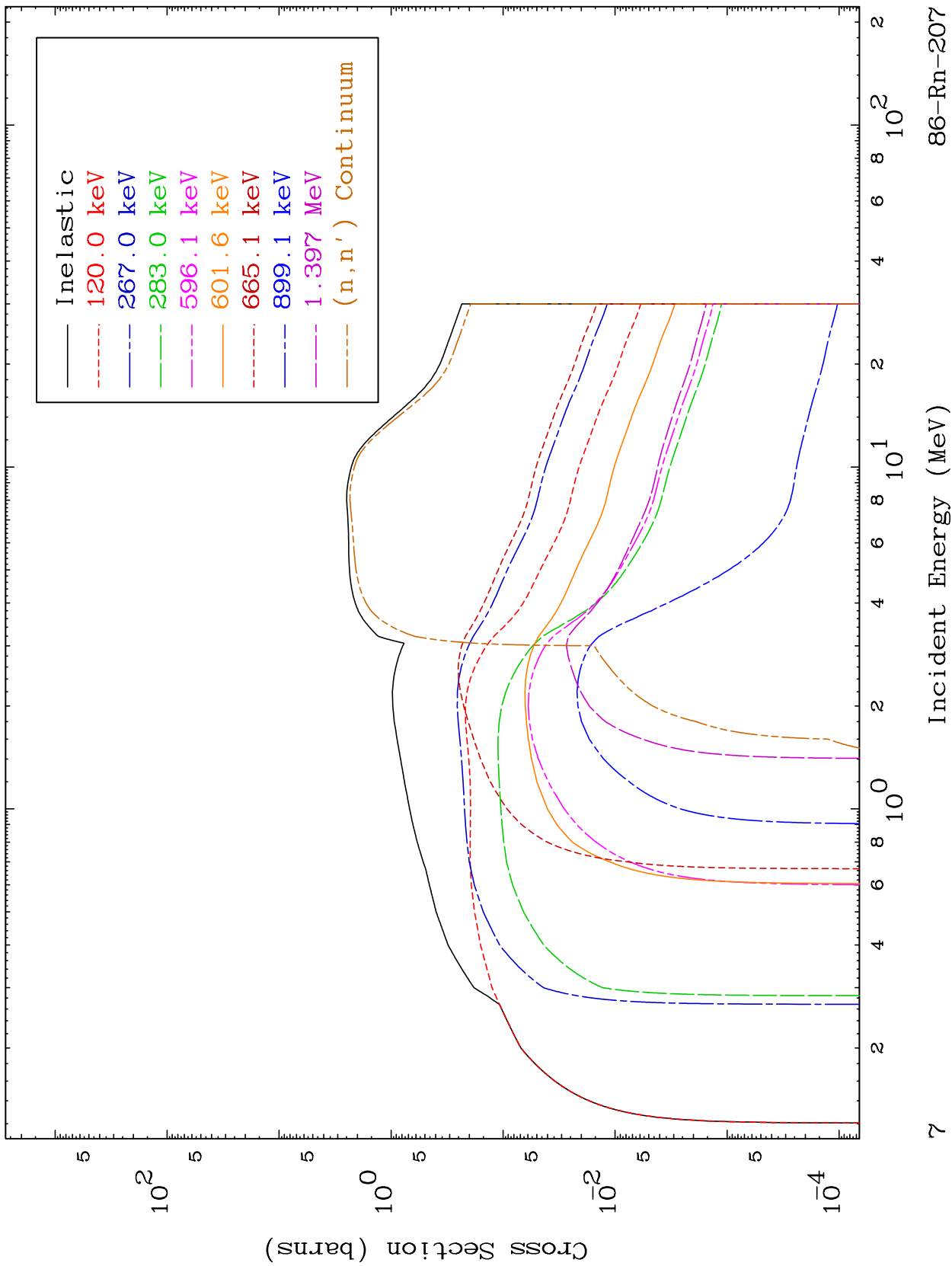
86-Rn-207

MAT 8613

Particle Production
293 Kelvin Cross Sections

86-Rn-207

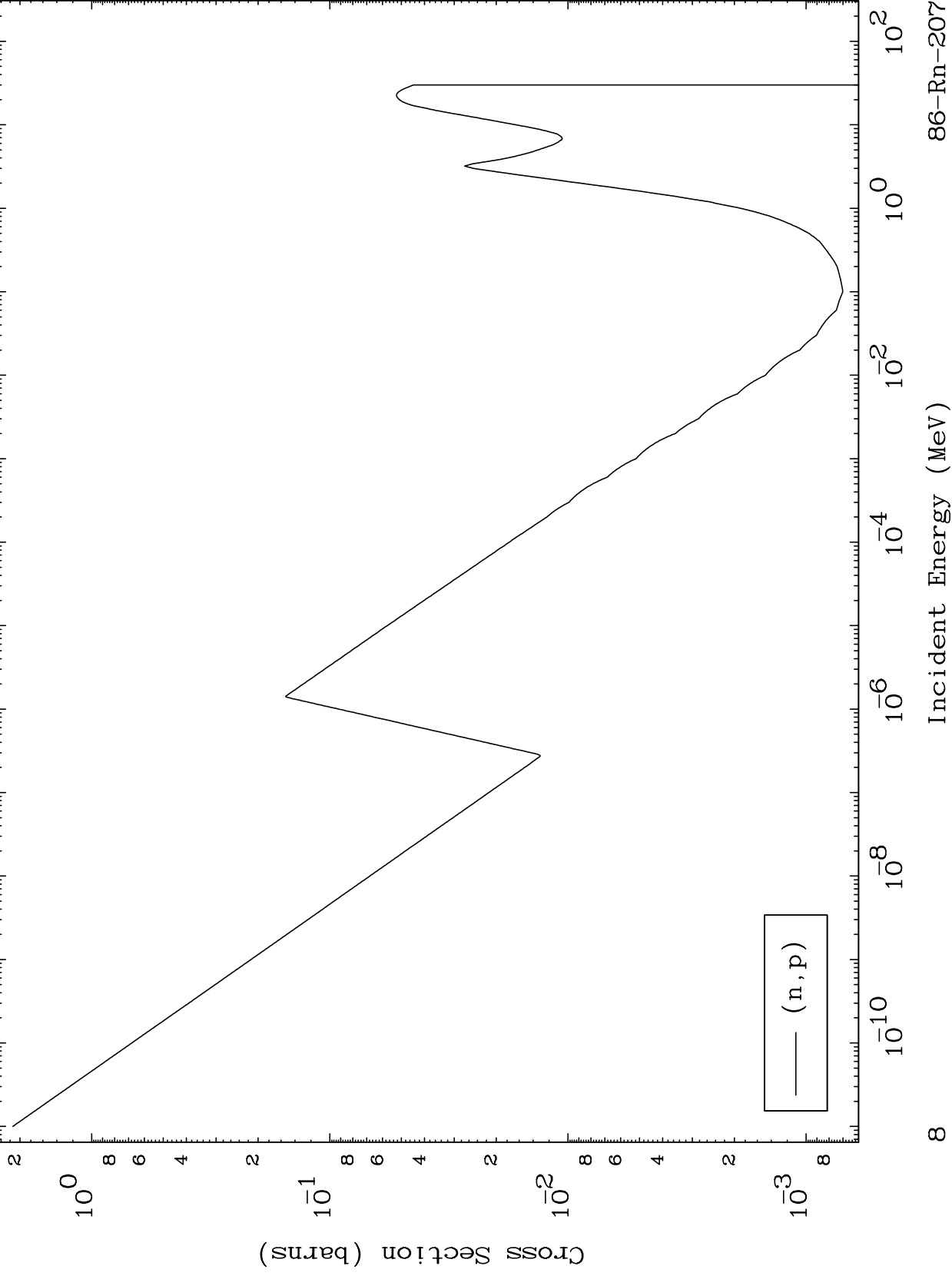




MAT 8613

(n,p) Levels
293 Kelvin Cross Sections

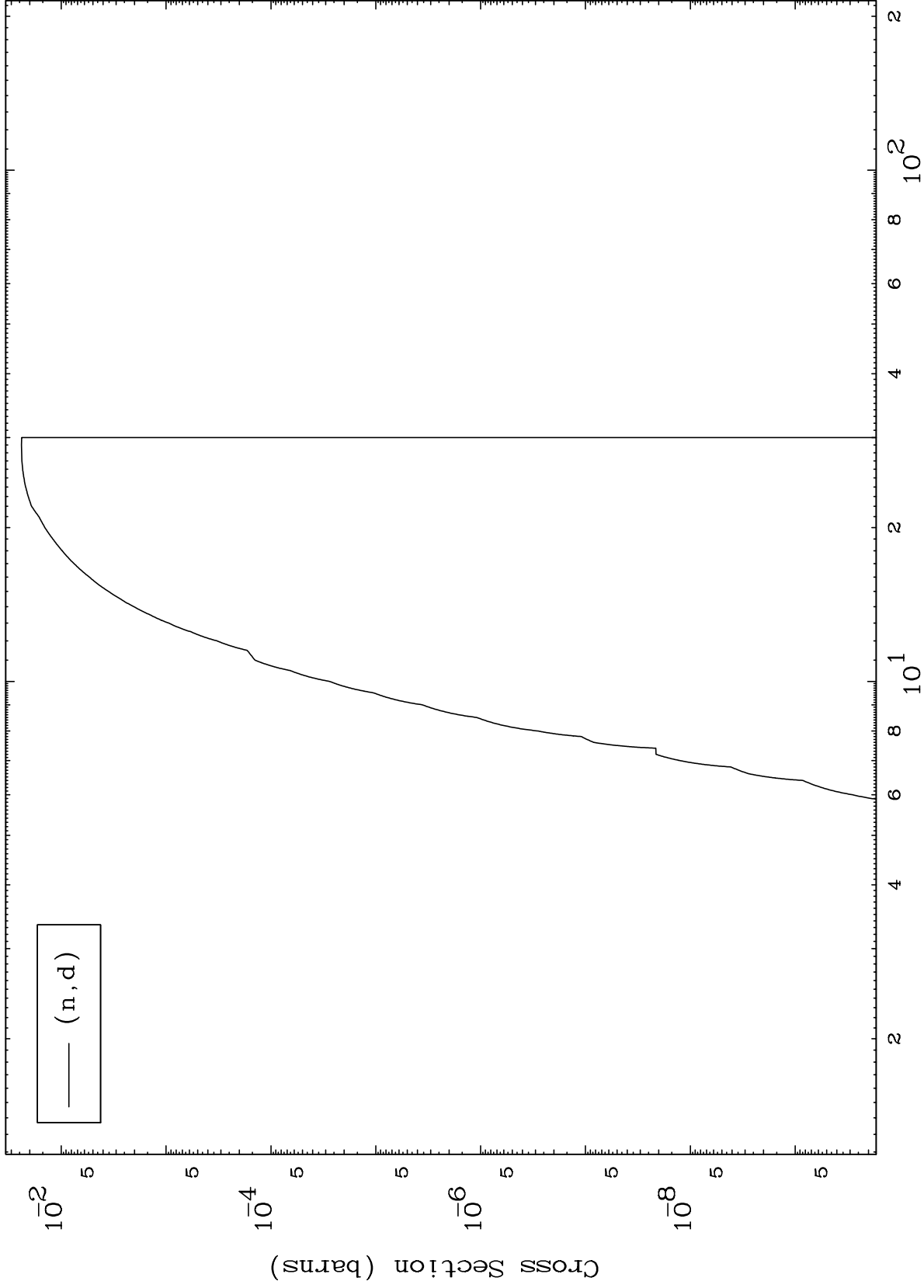
86-Rn-207



MAT 8613

(n,d) Levels
293 Kelvin Cross Sections

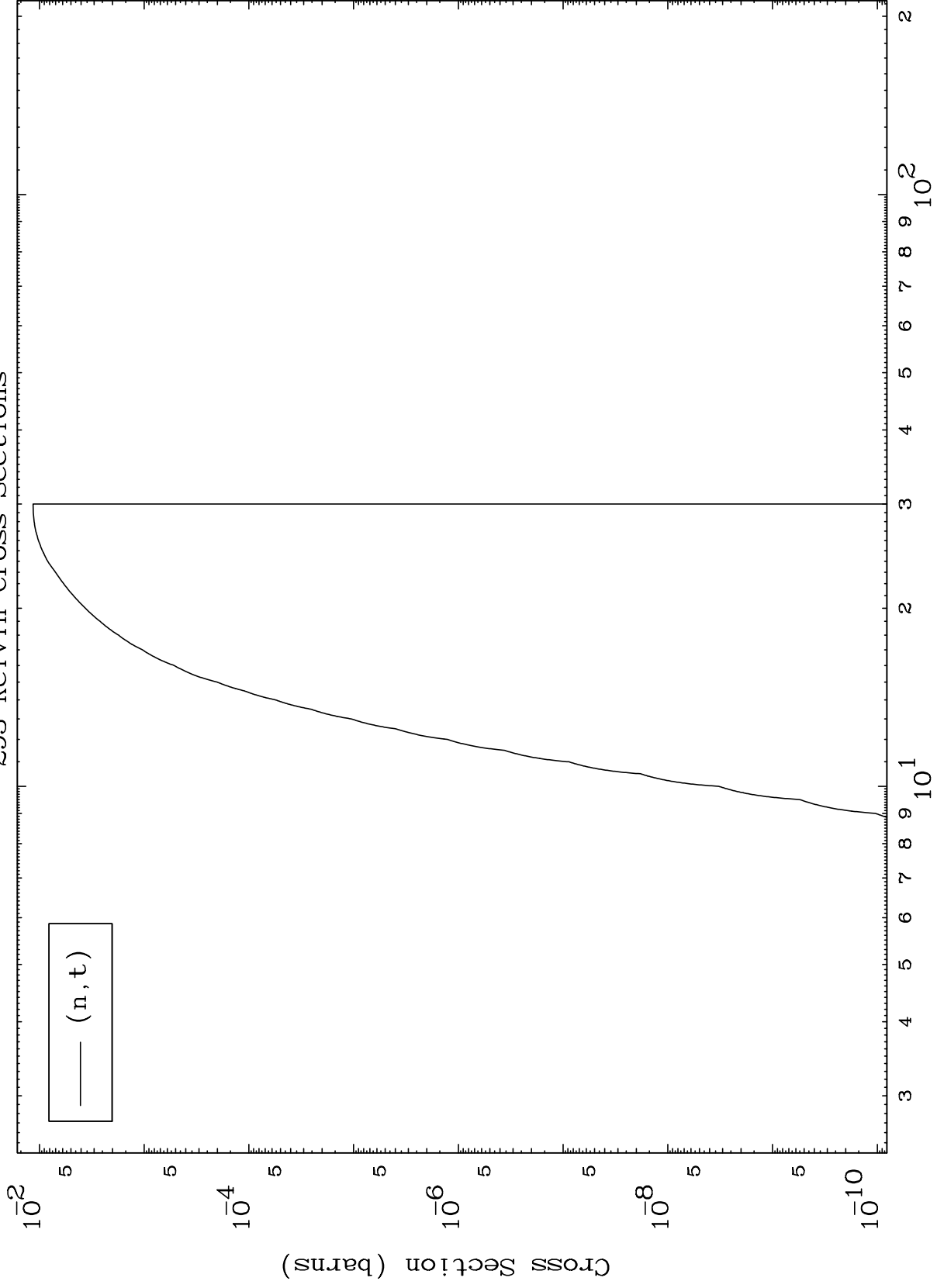
86-Rn-207



MAT 8613

(n,t) Levels
293 Kelvin Cross Sections

86-Rn-207



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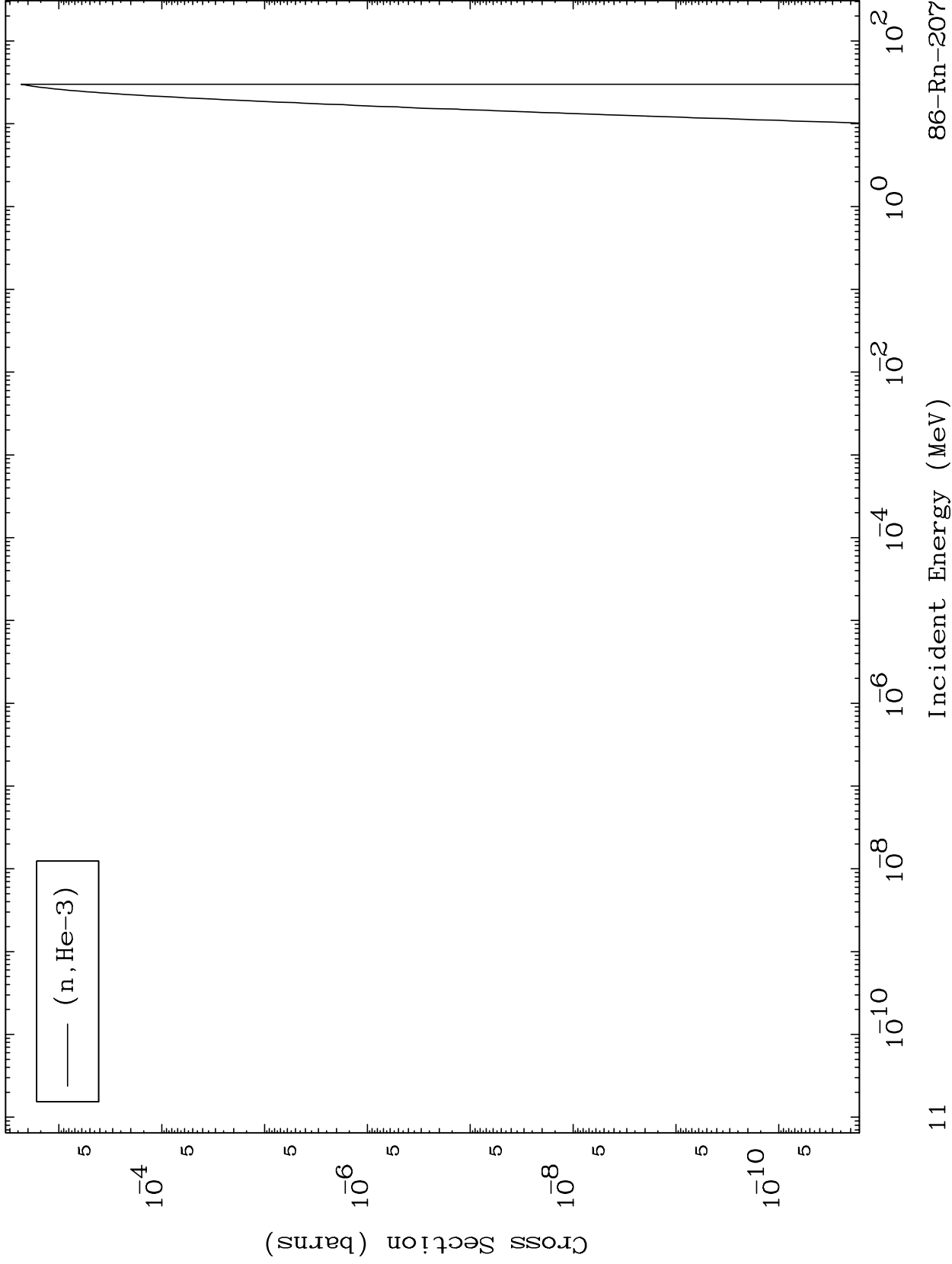
Incident Energy (MeV)

86-Rn-207

MAT 8613

(n,He3) Levels
293 Kelvin Cross Sections

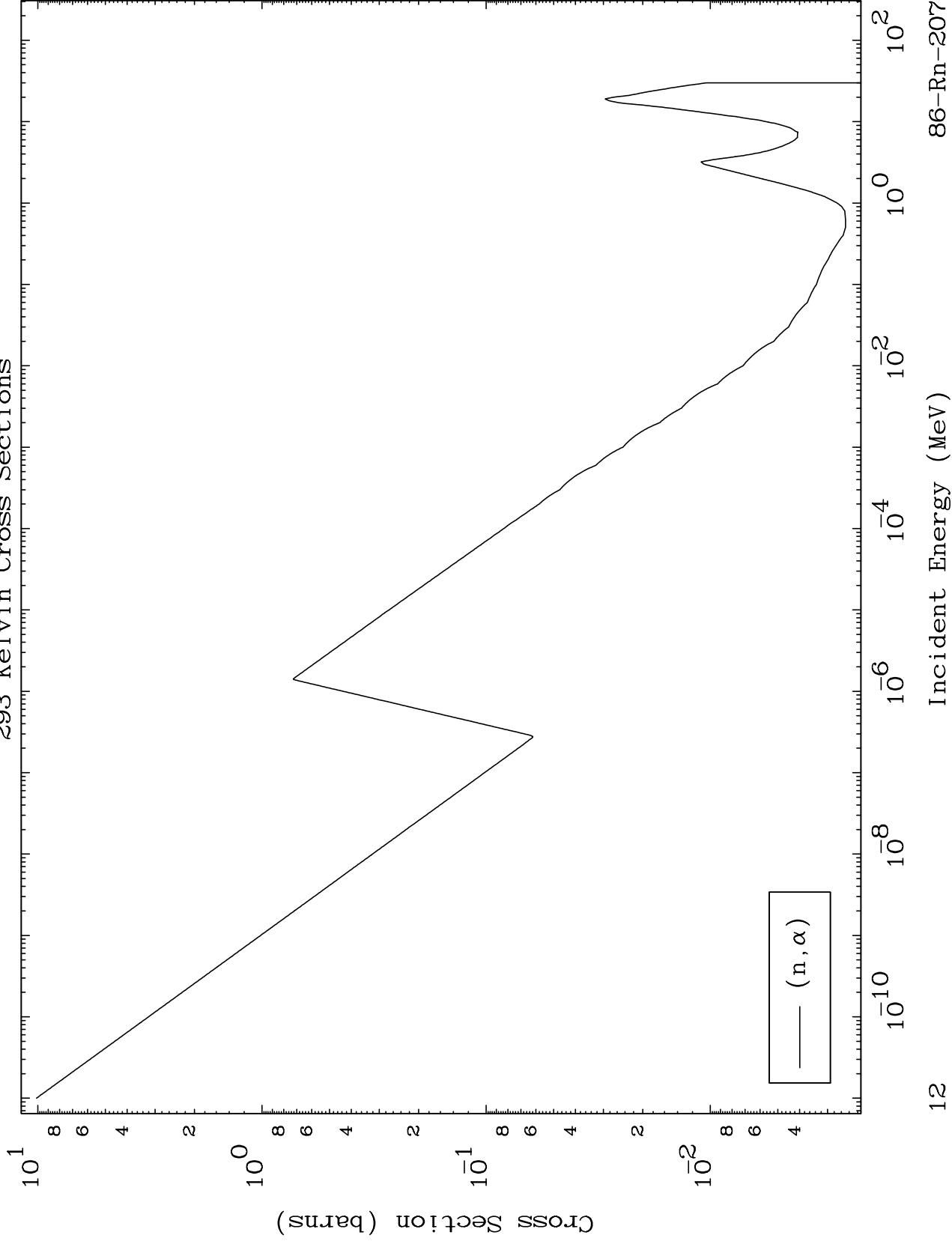
86-Rn-207

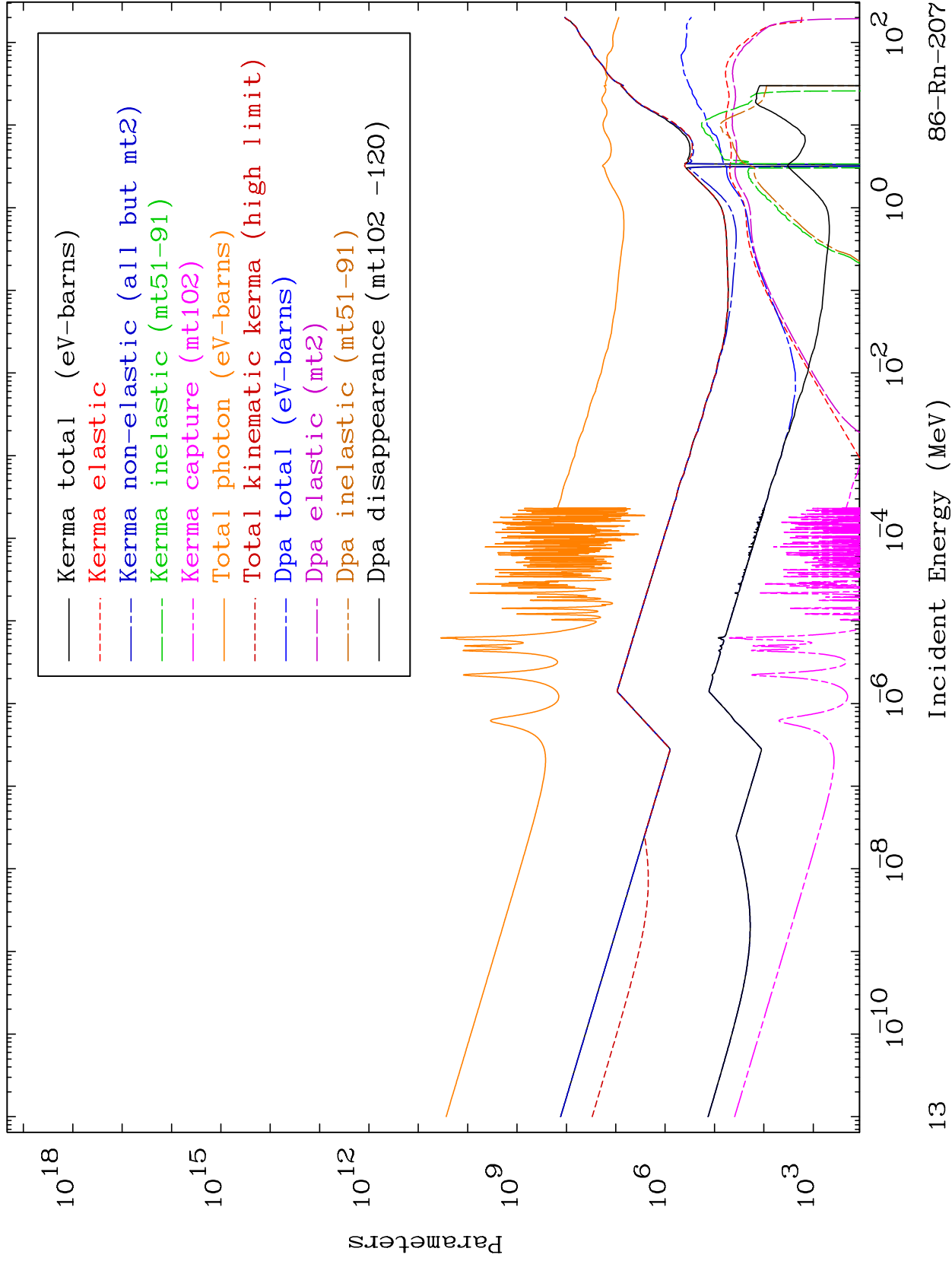


MAT 8613

(n,α) Levels
293 Kelvin Cross Sections

86-Rn-207

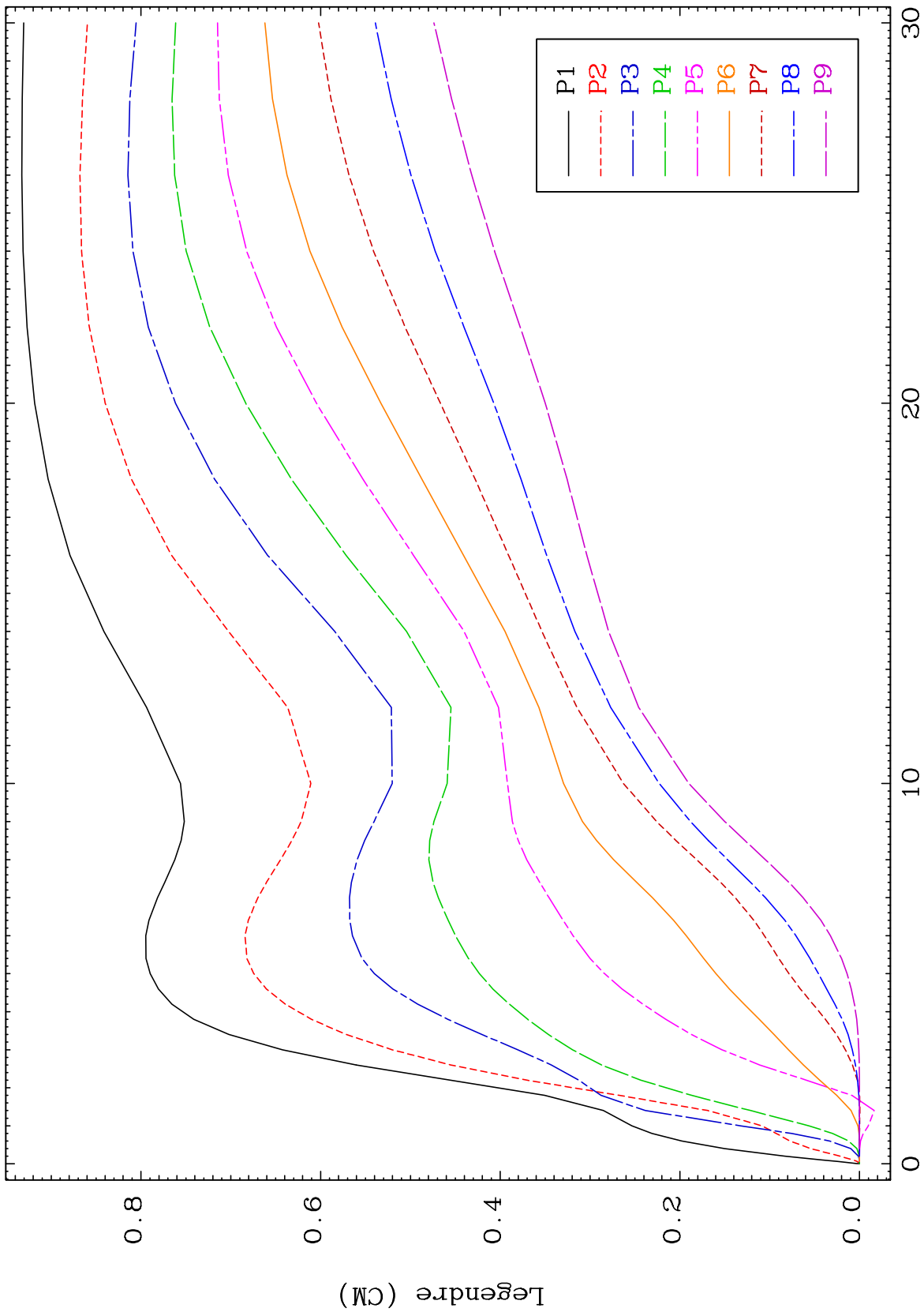




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86-Rn-207

Elastic Legendre Coefficients



86-Rn-207

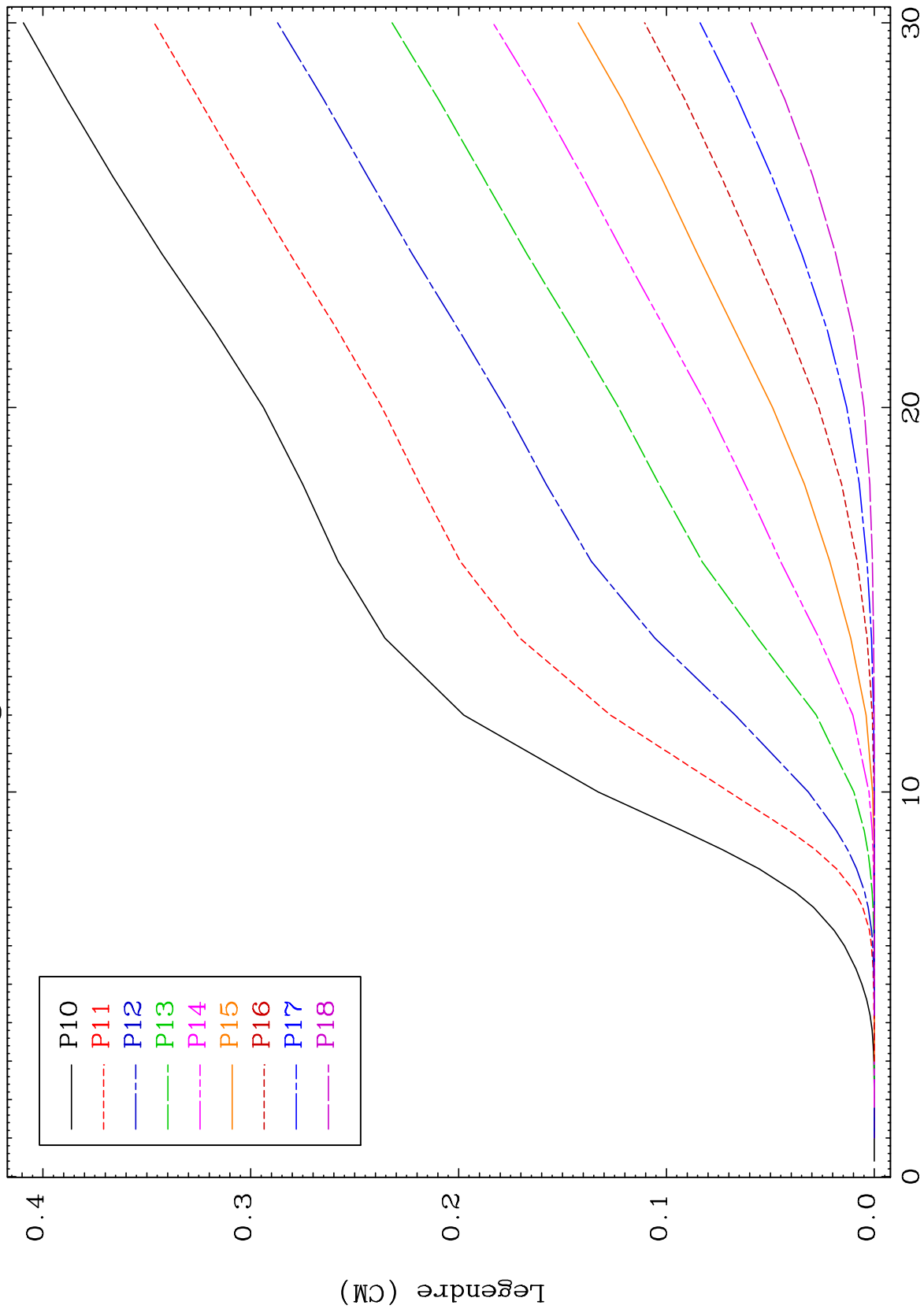
Incident Energy (MeV)

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MAT 8613

Elastic Legendre Coefficients

86-Rn-207



15

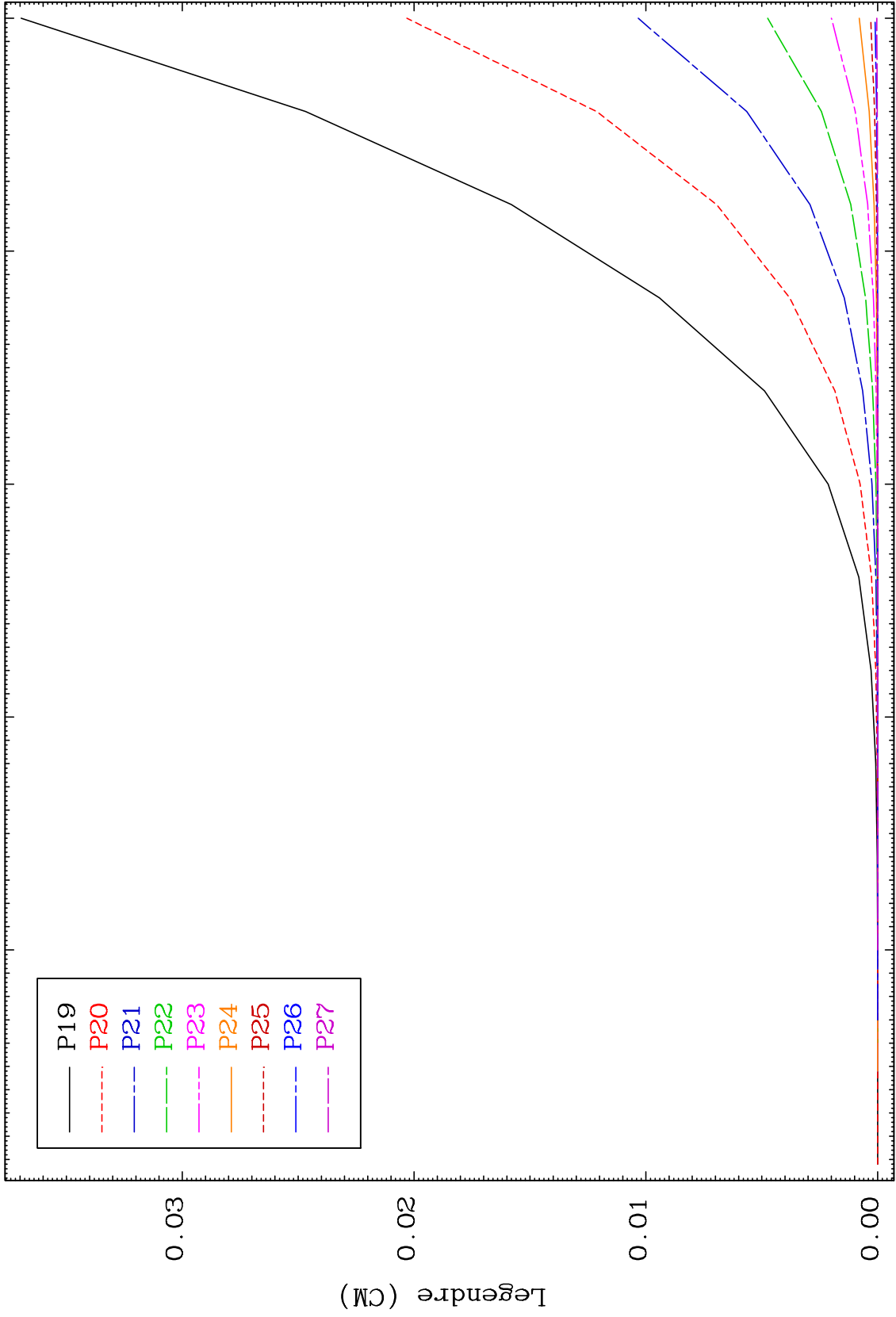
Incident Energy (MeV)

86-Rn-207

MAT 8613

Elastic Legendre Coefficients

86-Rn-207



16

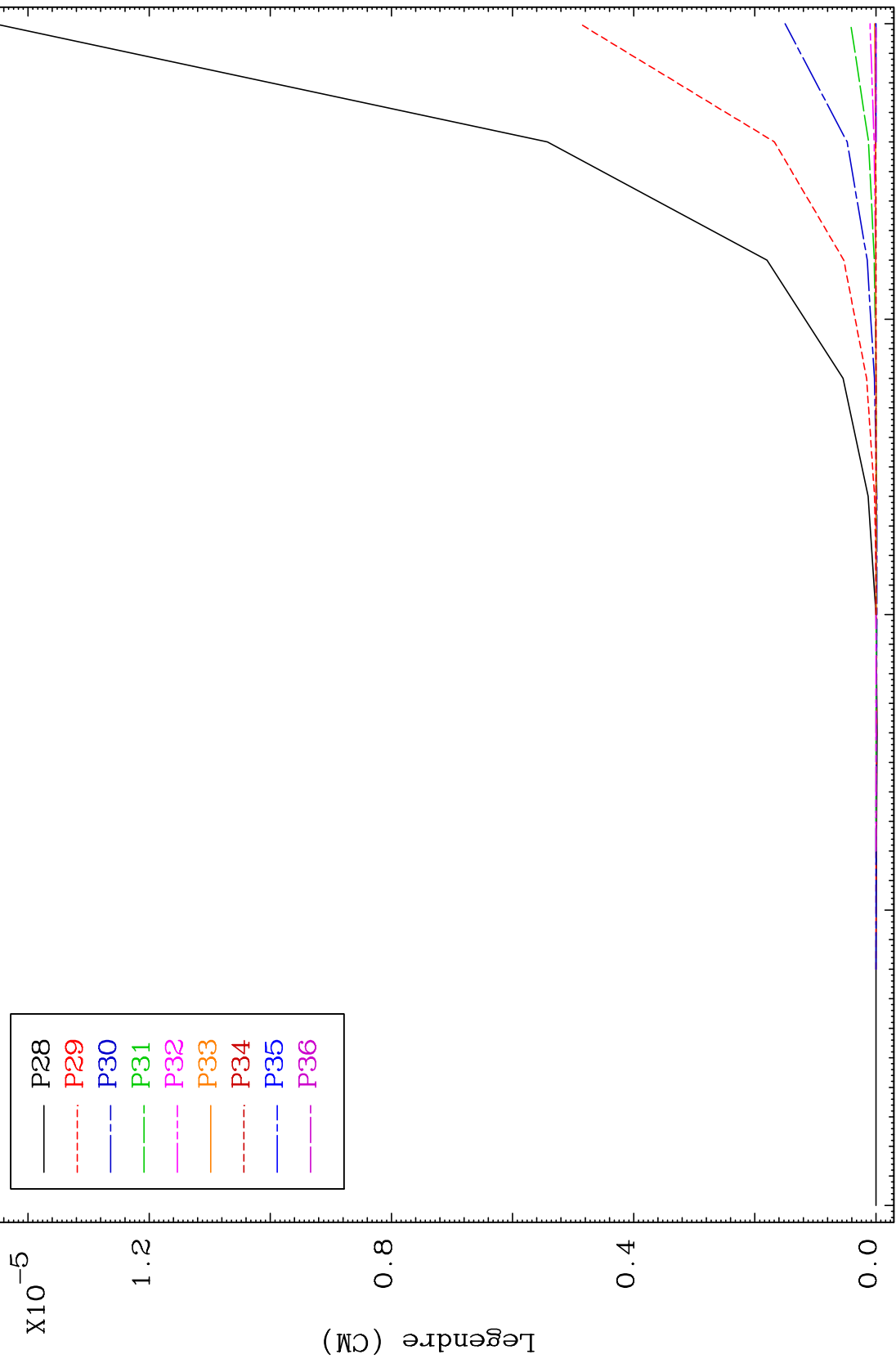
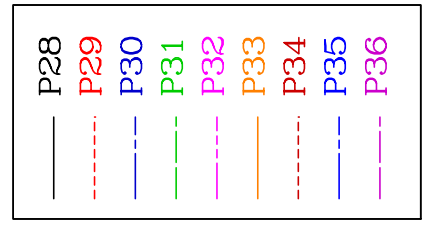
Incident Energy (MeV)

86-Rn-207

MAT 8613

Elastic Legendre Coefficients

86-Rn-207



17

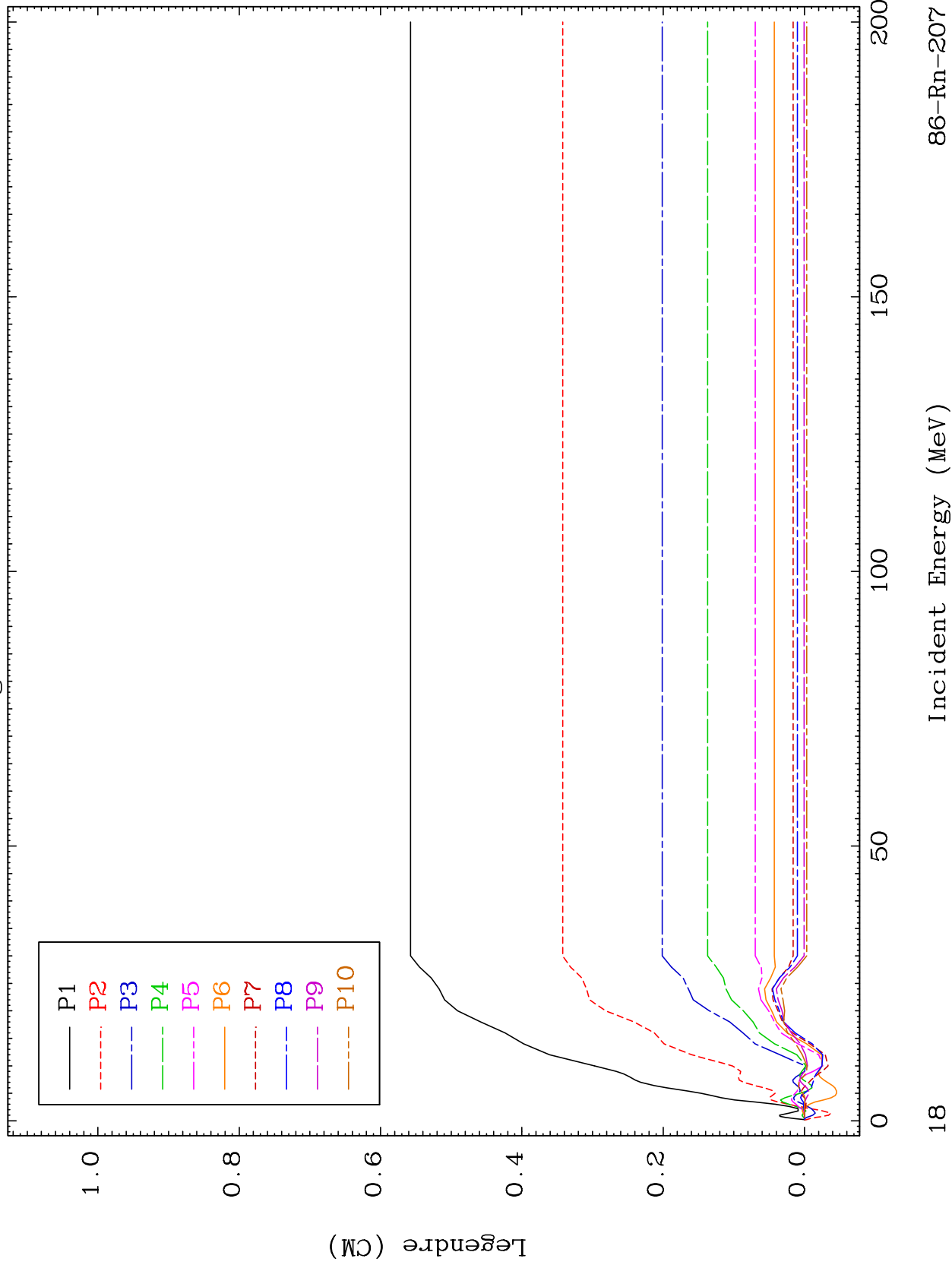
Incident Energy (MeV)

86-Rn-207

MAT 8613

120.0 keV (n,n') Level
Legendre Coefficients

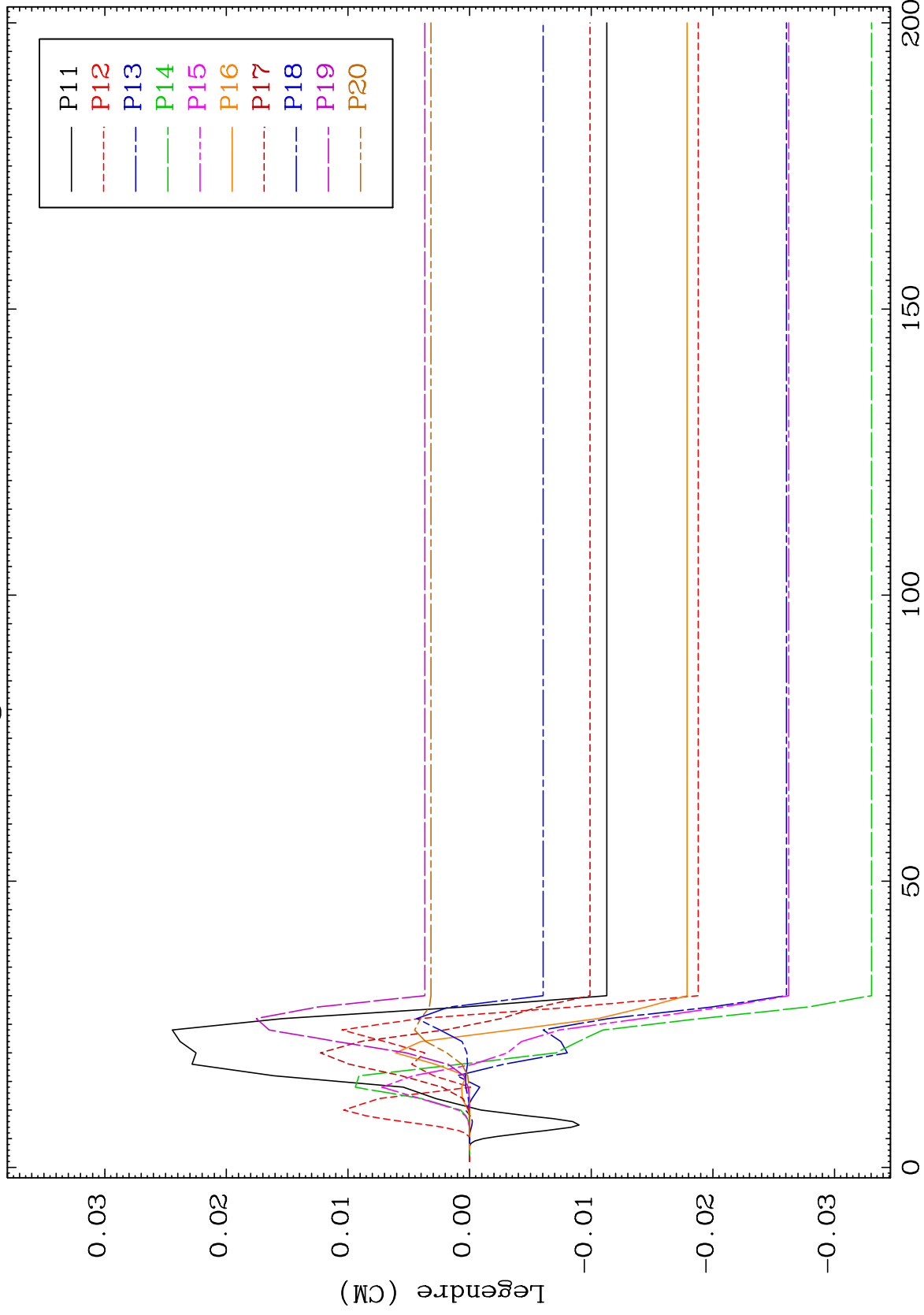
86-Rn-207



MAT 8613

120.0 keV (n,n') Level
Legendre Coefficients

86-Rn-207



19

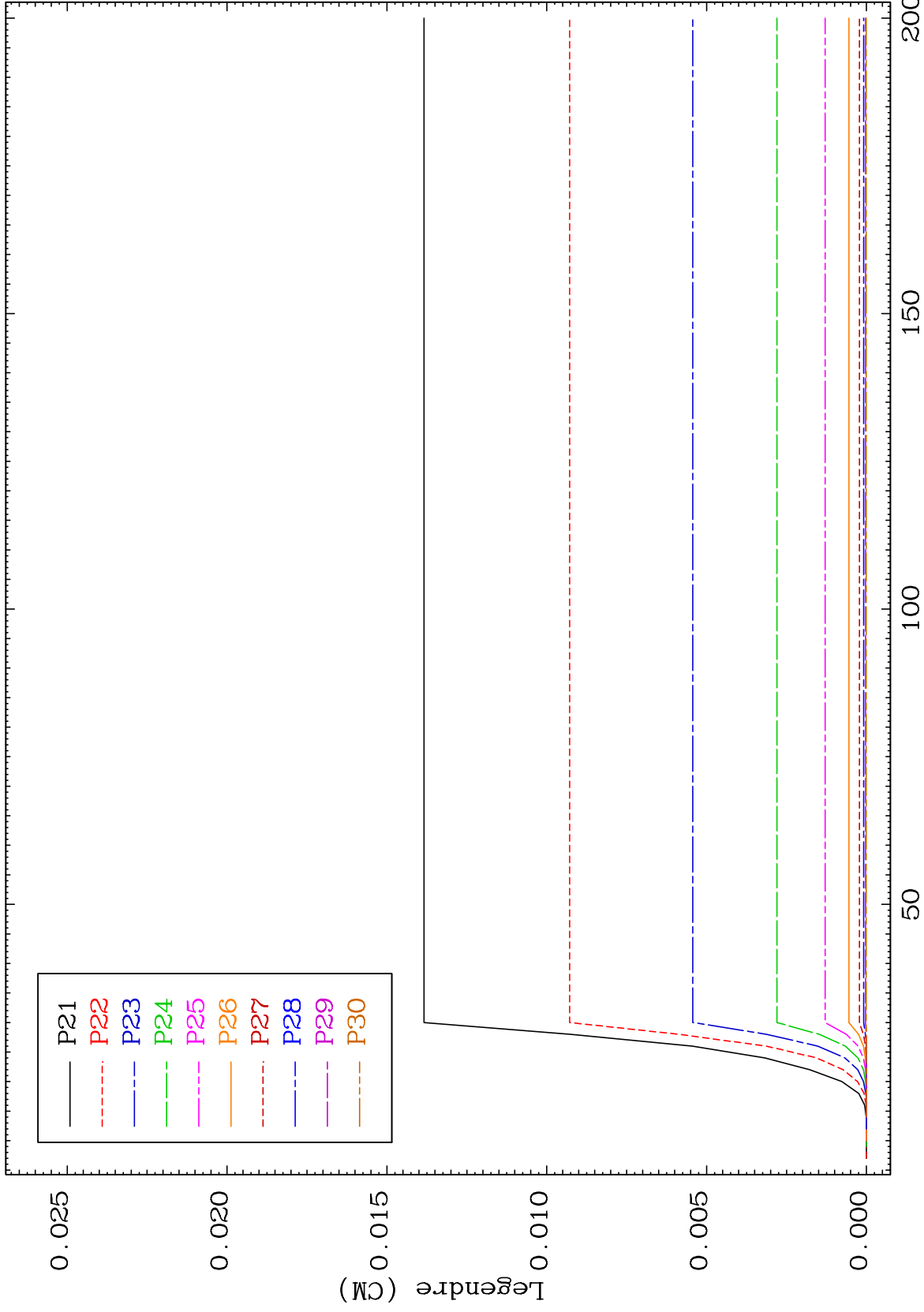
Incident Energy (MeV)

86-Rn-207

MAT 8613

120.0 keV (n,n') Level
Legendre Coefficients

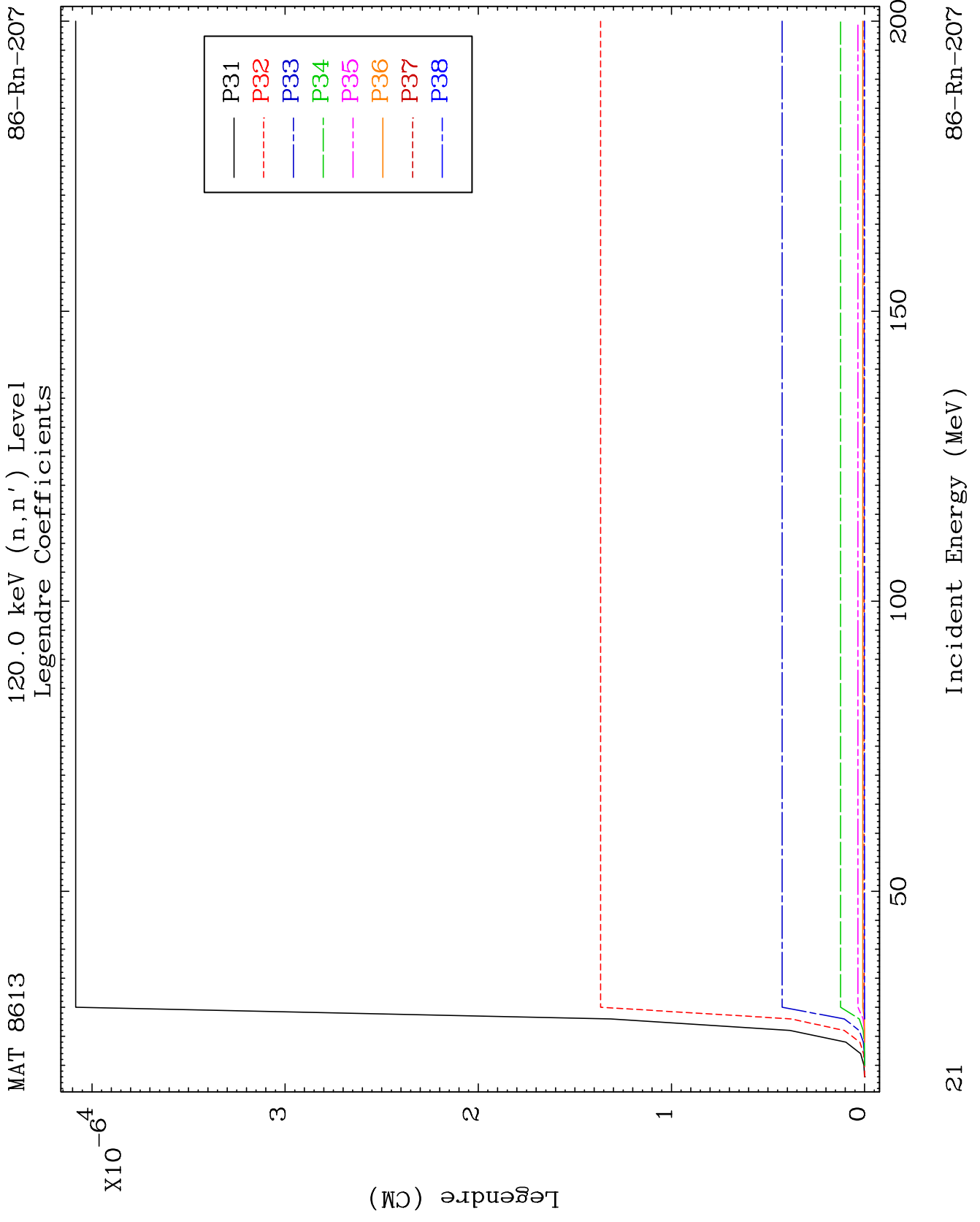
86-Rn-207



20

Incident Energy (MeV)

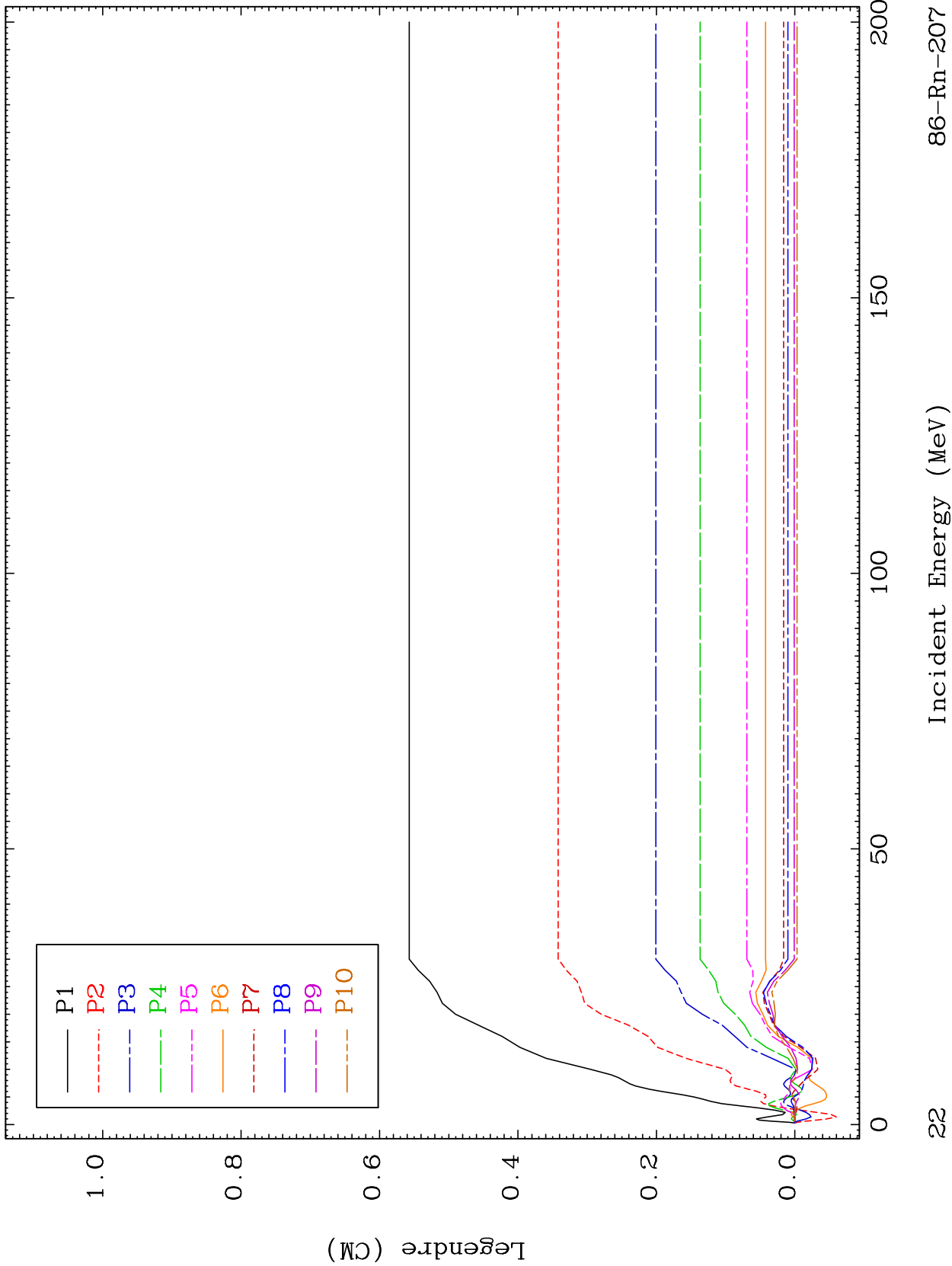
86-Rn-207



MAT 8613

267.0 keV (n,n') Level
Legendre Coefficients

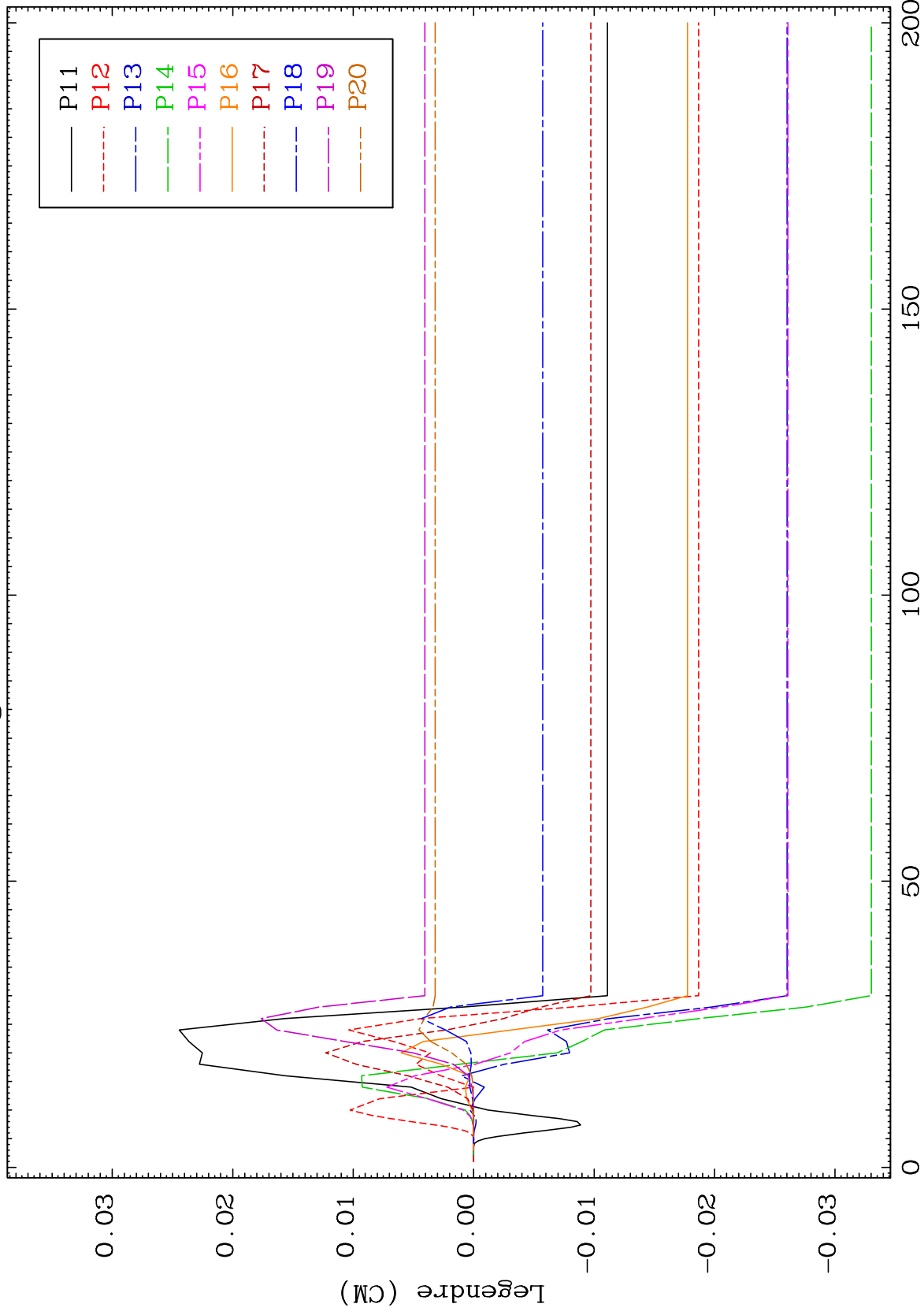
86-Rn-207



MAT 8613

267.0 keV (n,n') Level
Legendre Coefficients

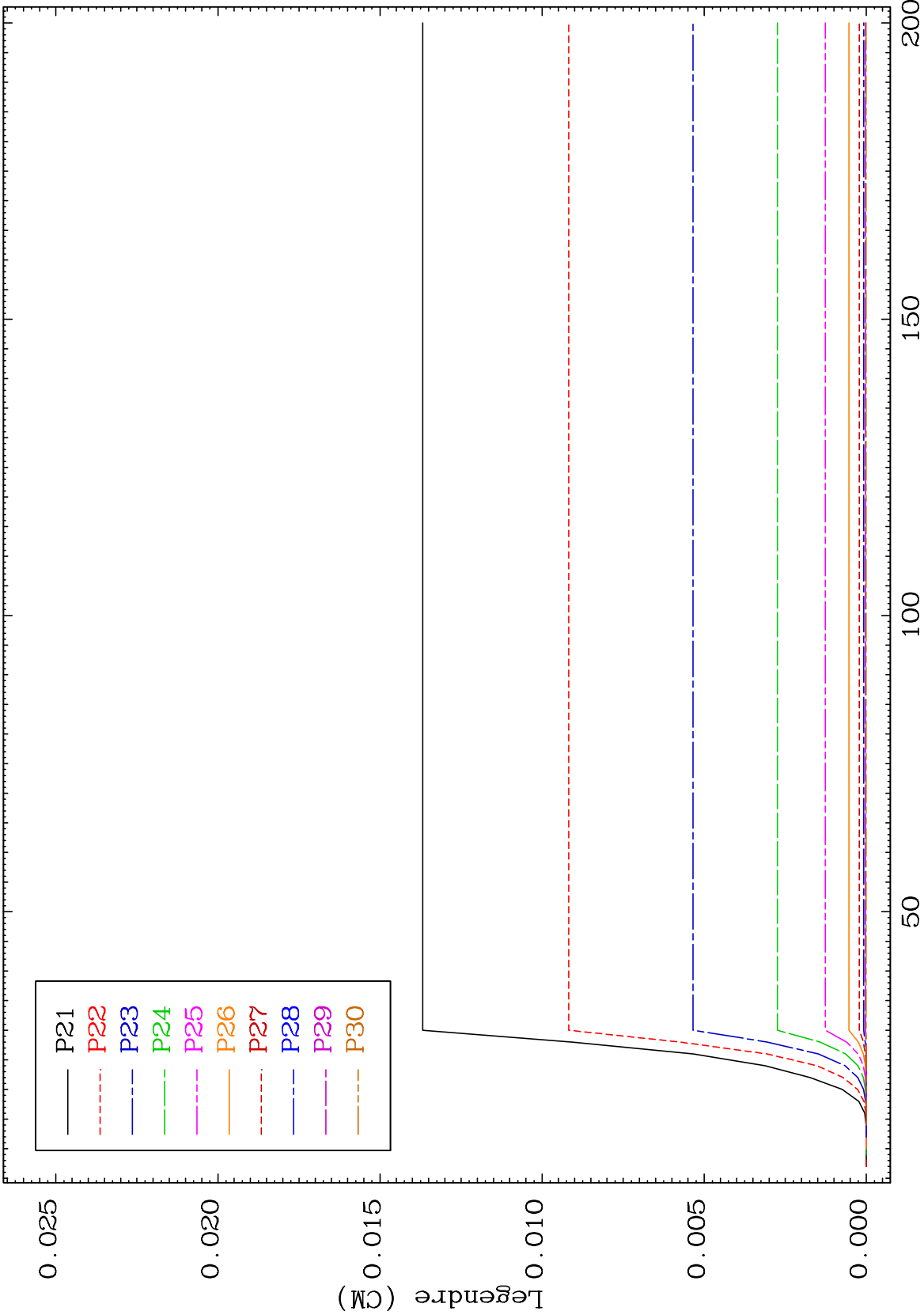
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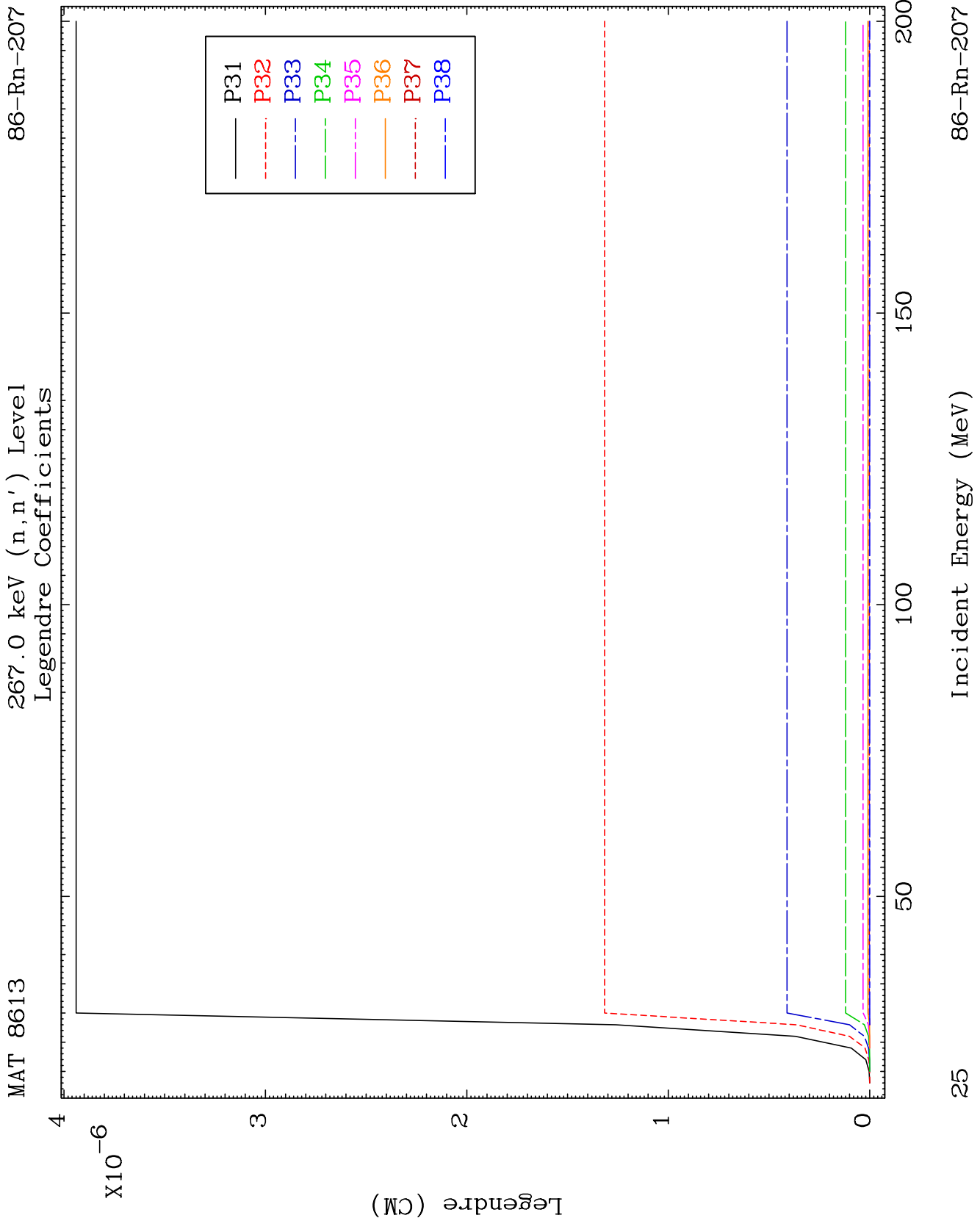


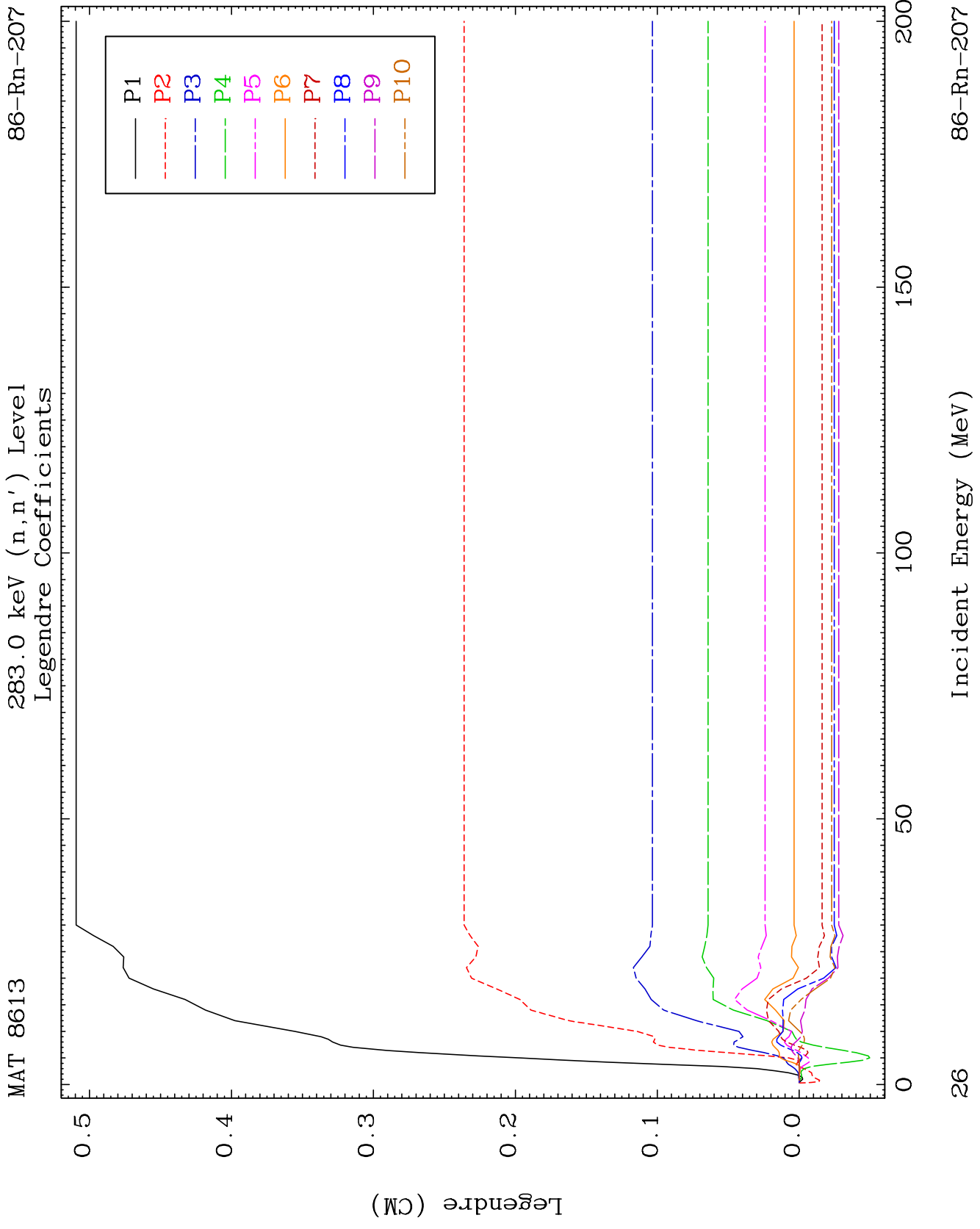
23

Incident Energy (MeV)

86-Rn-207



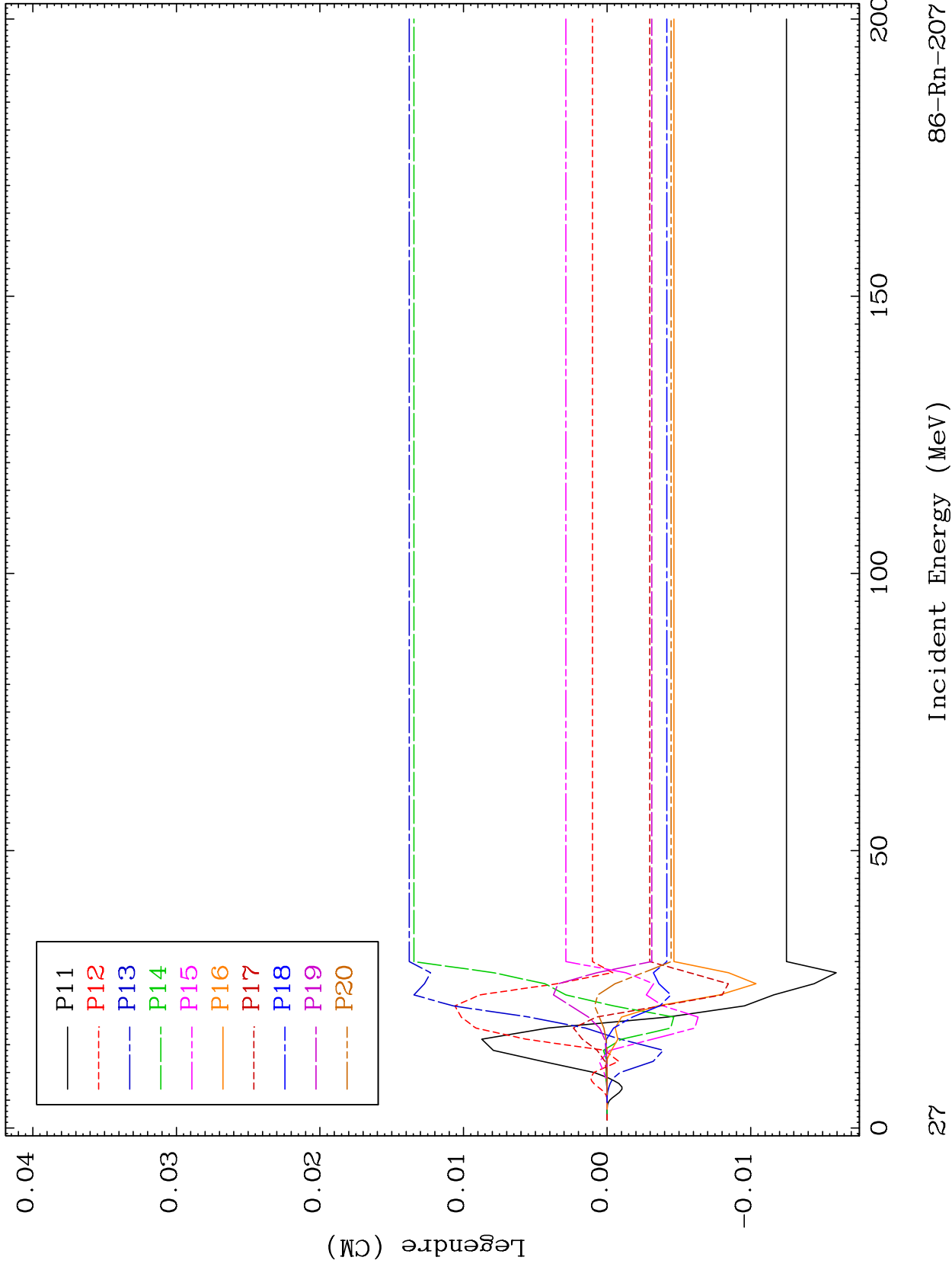




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283.0 keV (n,n') Level
Legendre Coefficients

86-Rn-207



27

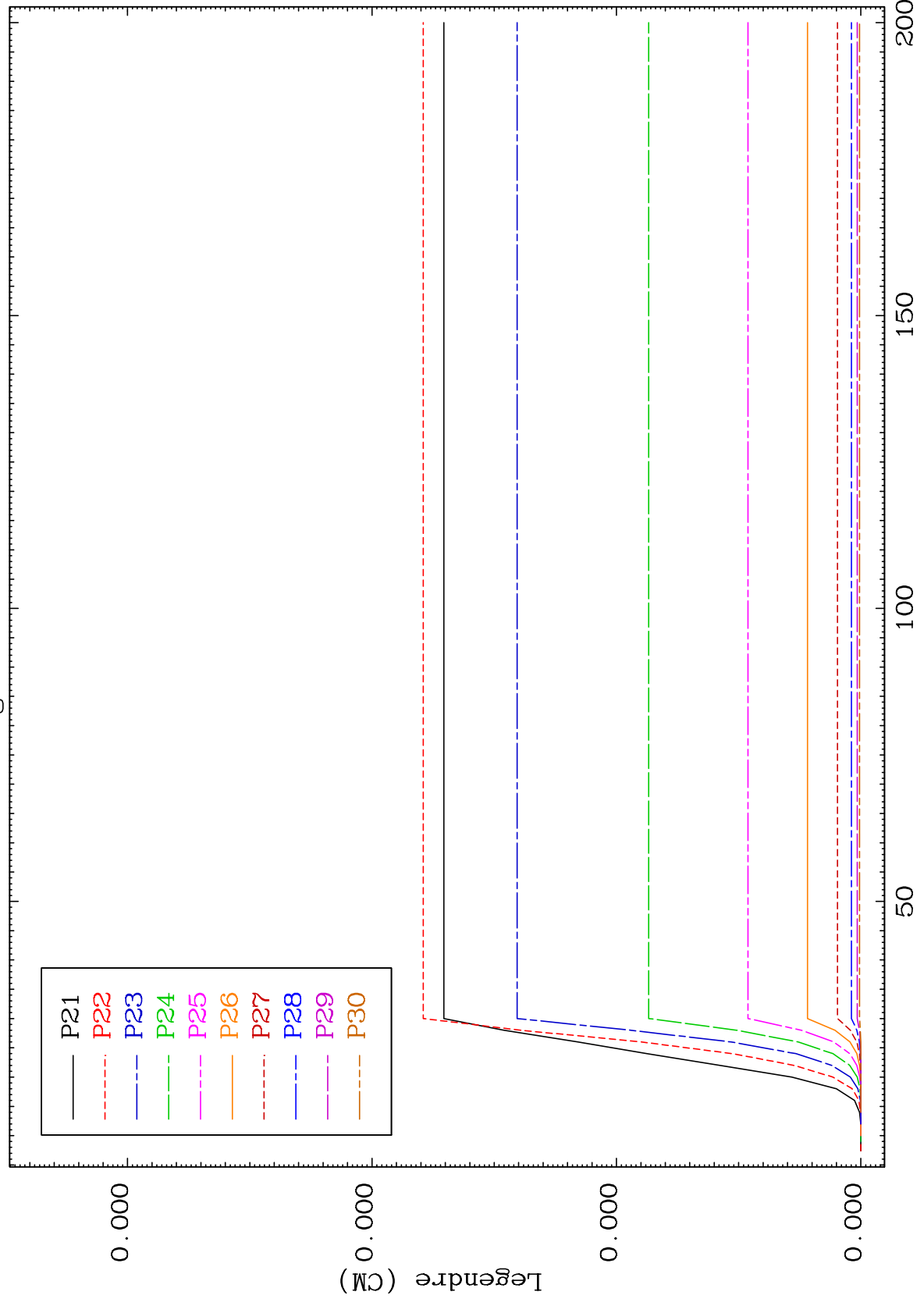
Incident Energy (MeV)

86-Rn-207

MAT 8613

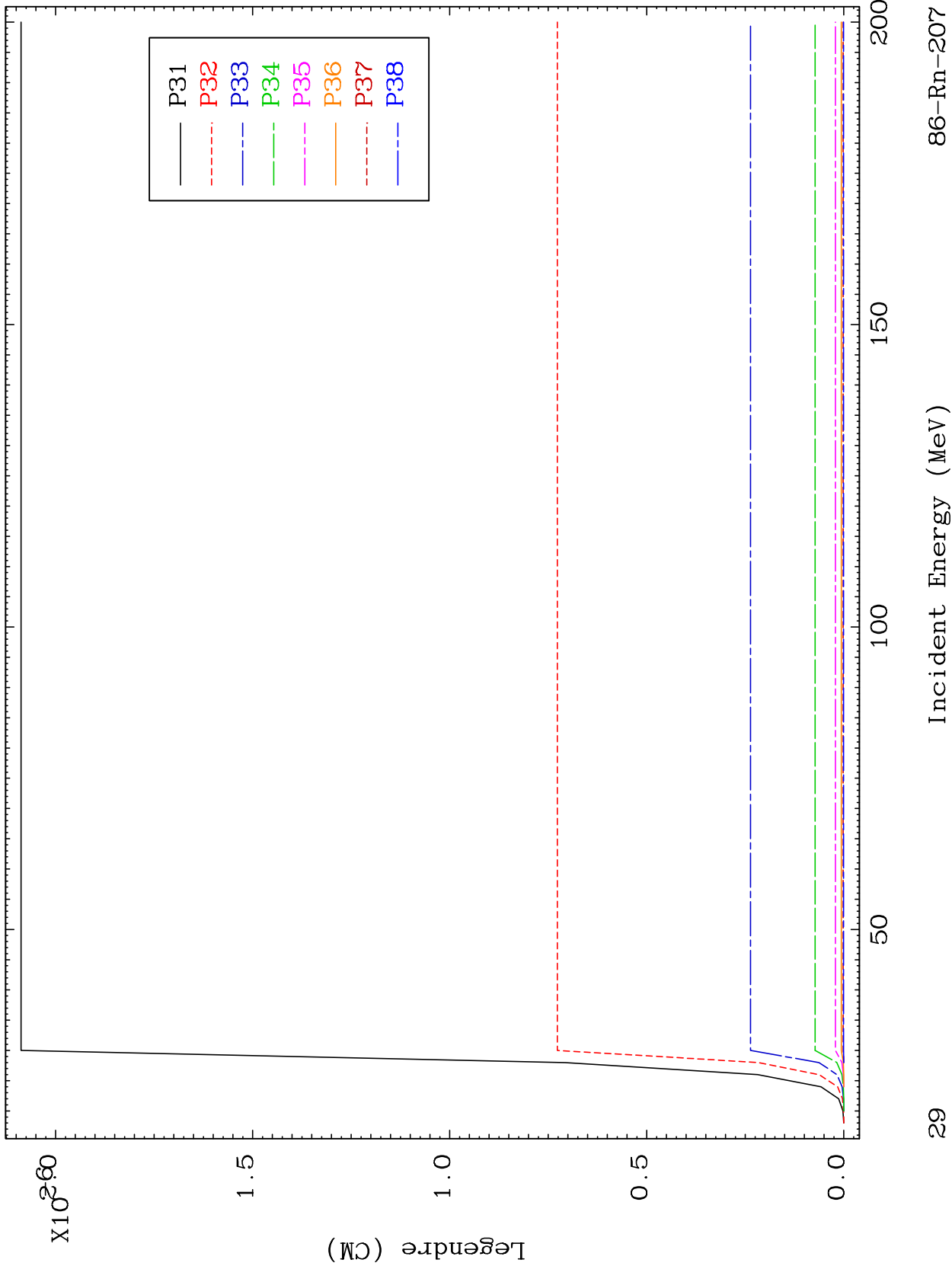
283.0 keV (n, n') Level
Legendre Coefficients

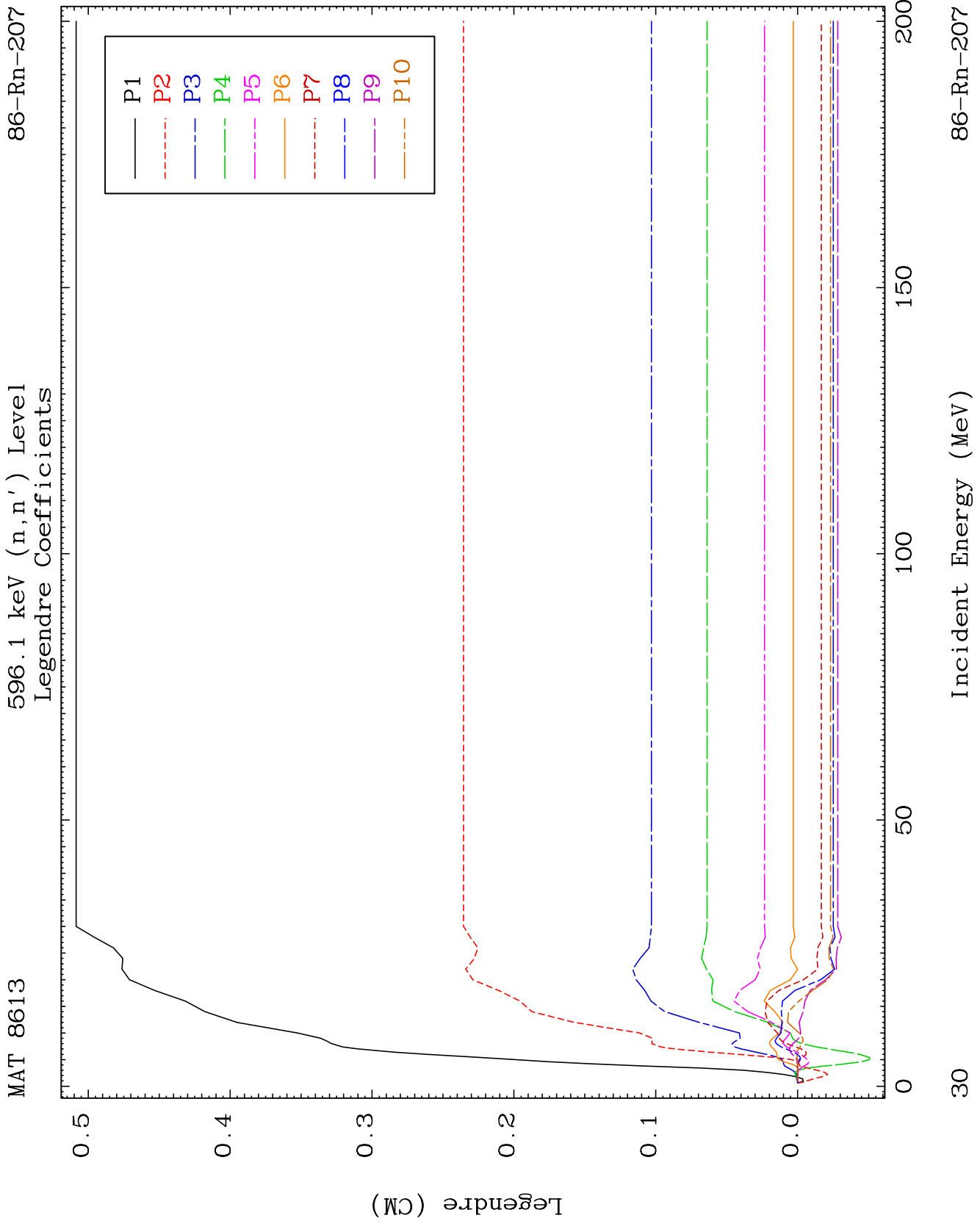
86-Rn-207



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86-Rn-207

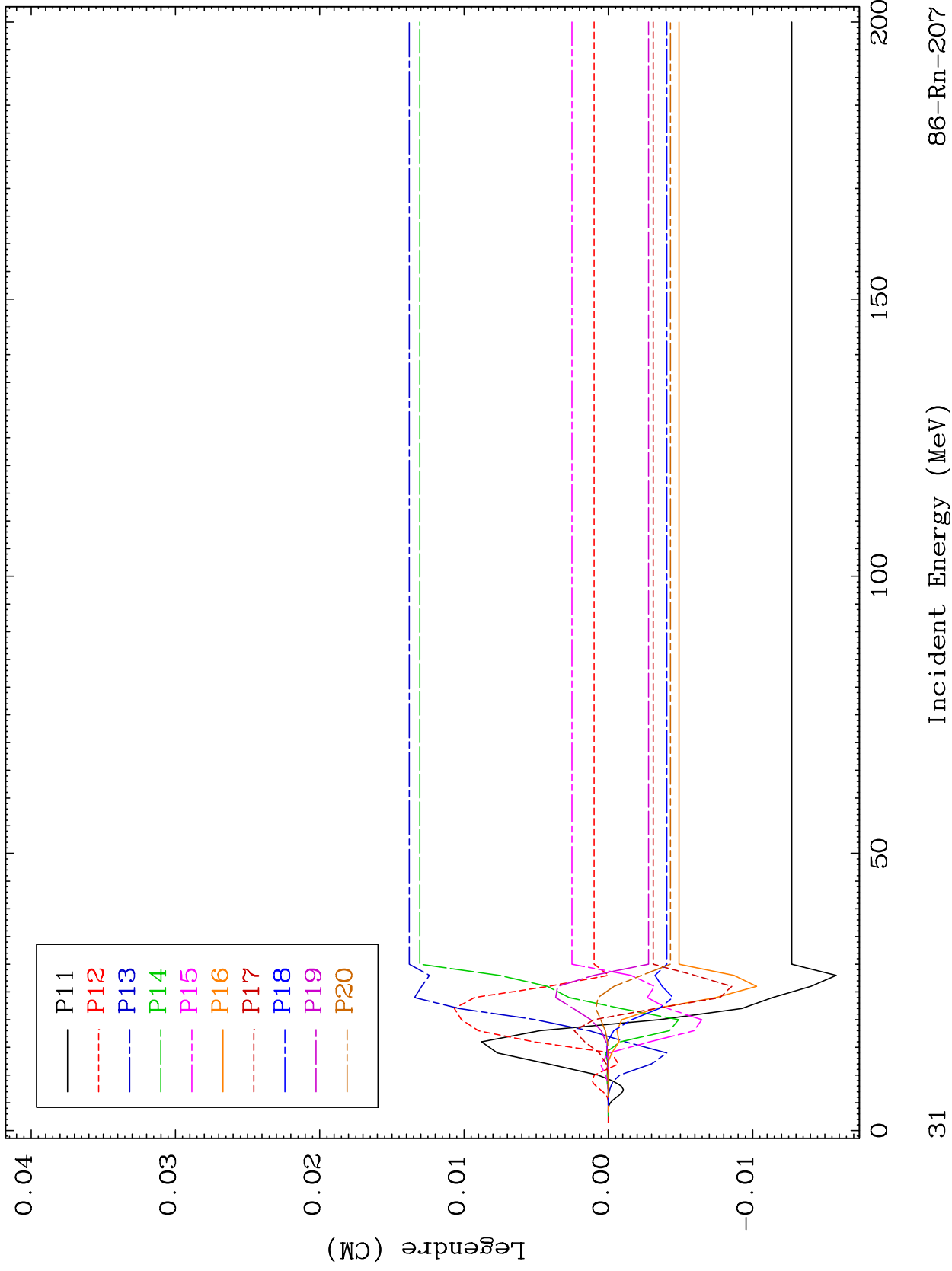




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596.1 keV (n,n') Level
Legendre Coefficients

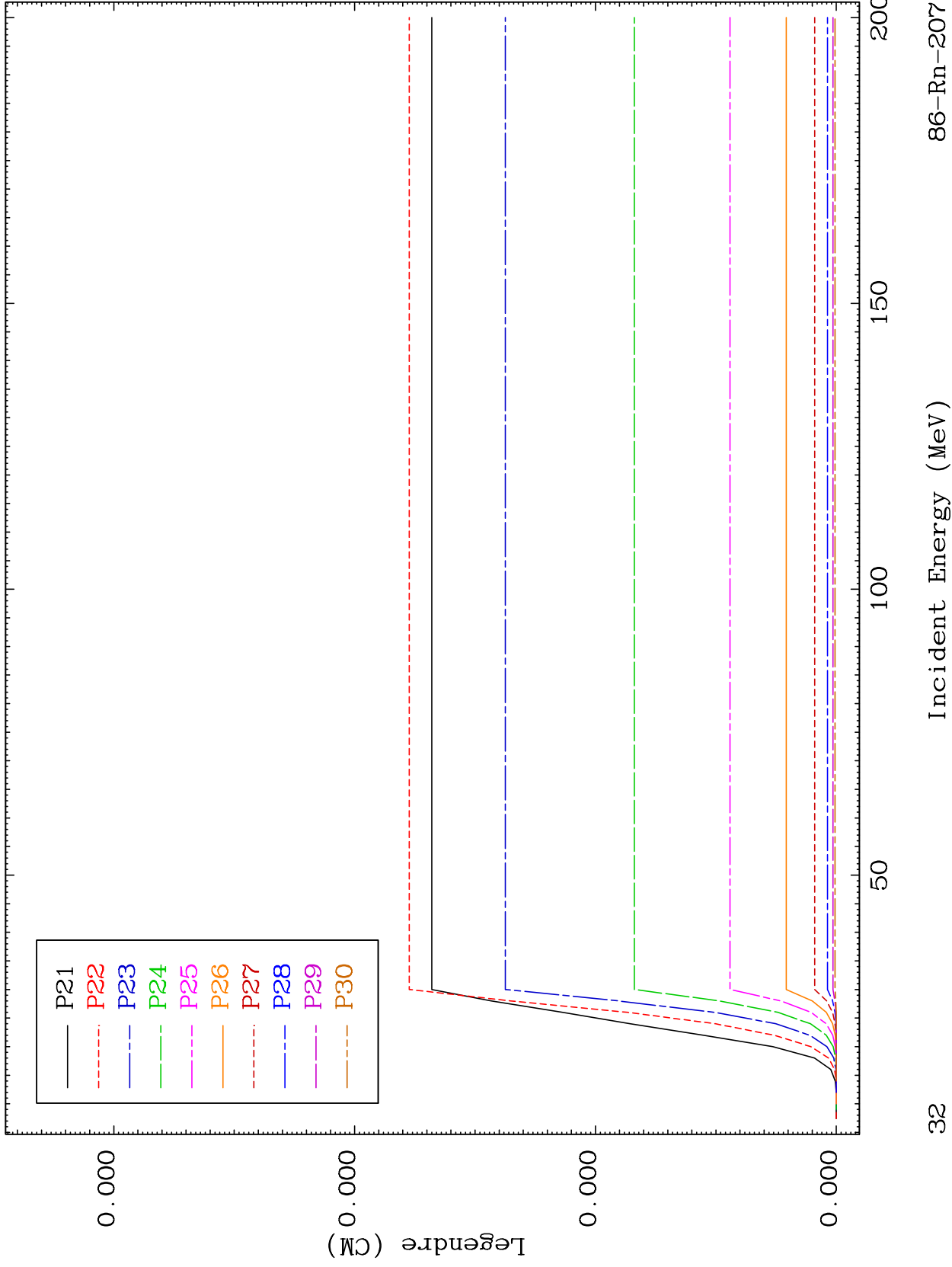
86-Rn-207

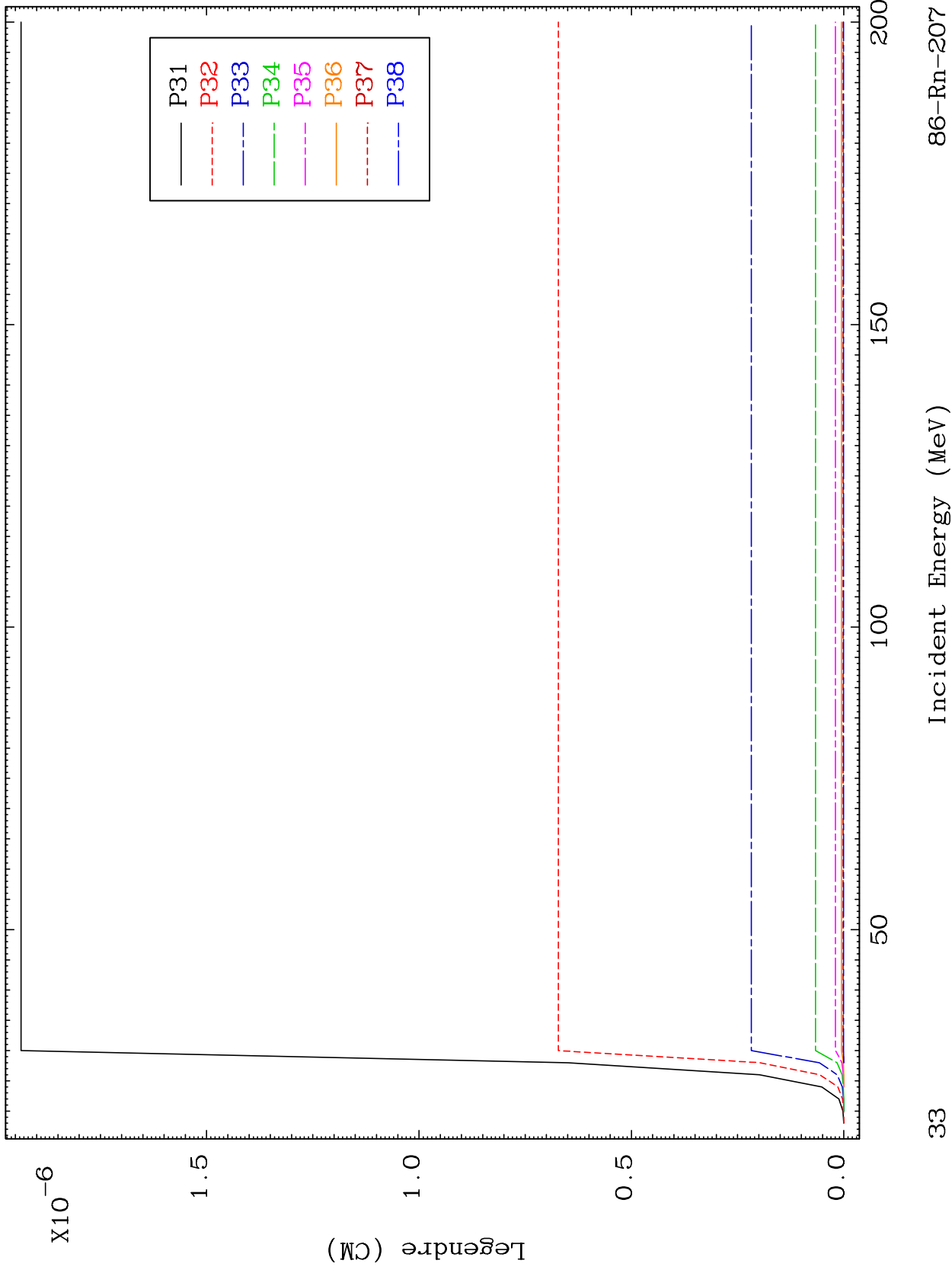


31

Incident Energy (MeV)

86-Rn-207

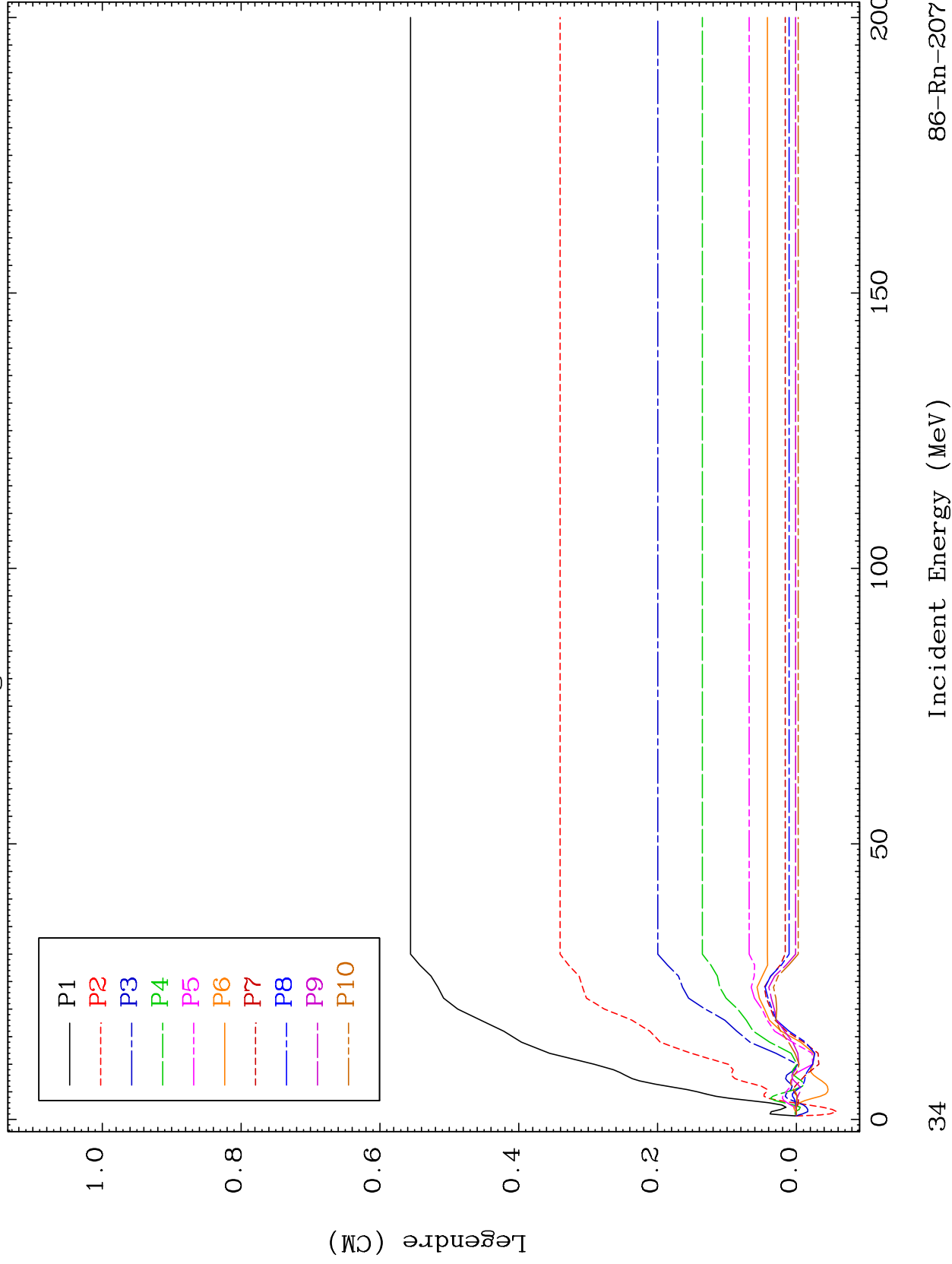


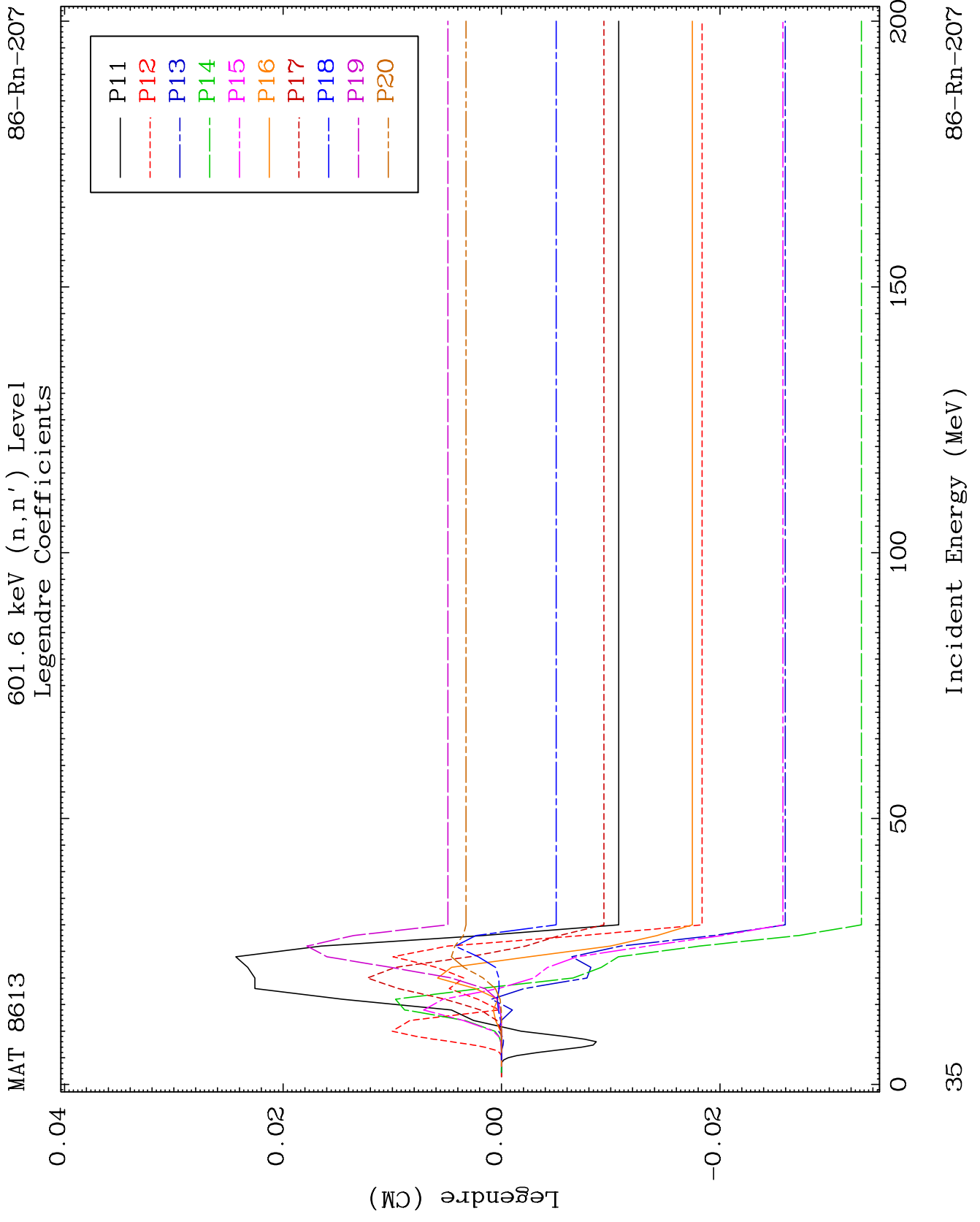


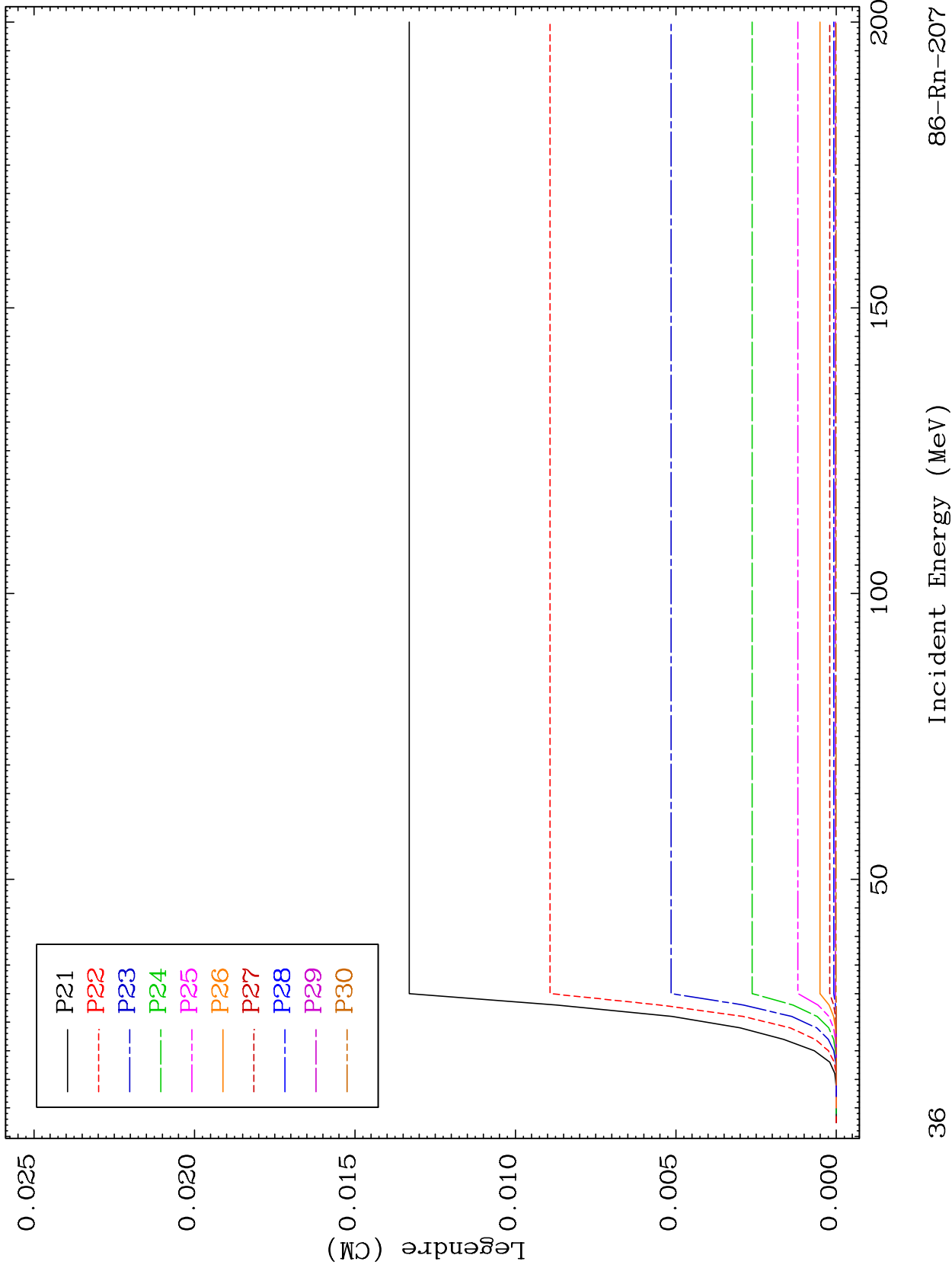
MAT 8613

601.6 keV (n,n') Level
Legendre Coefficients

86-Rn-207



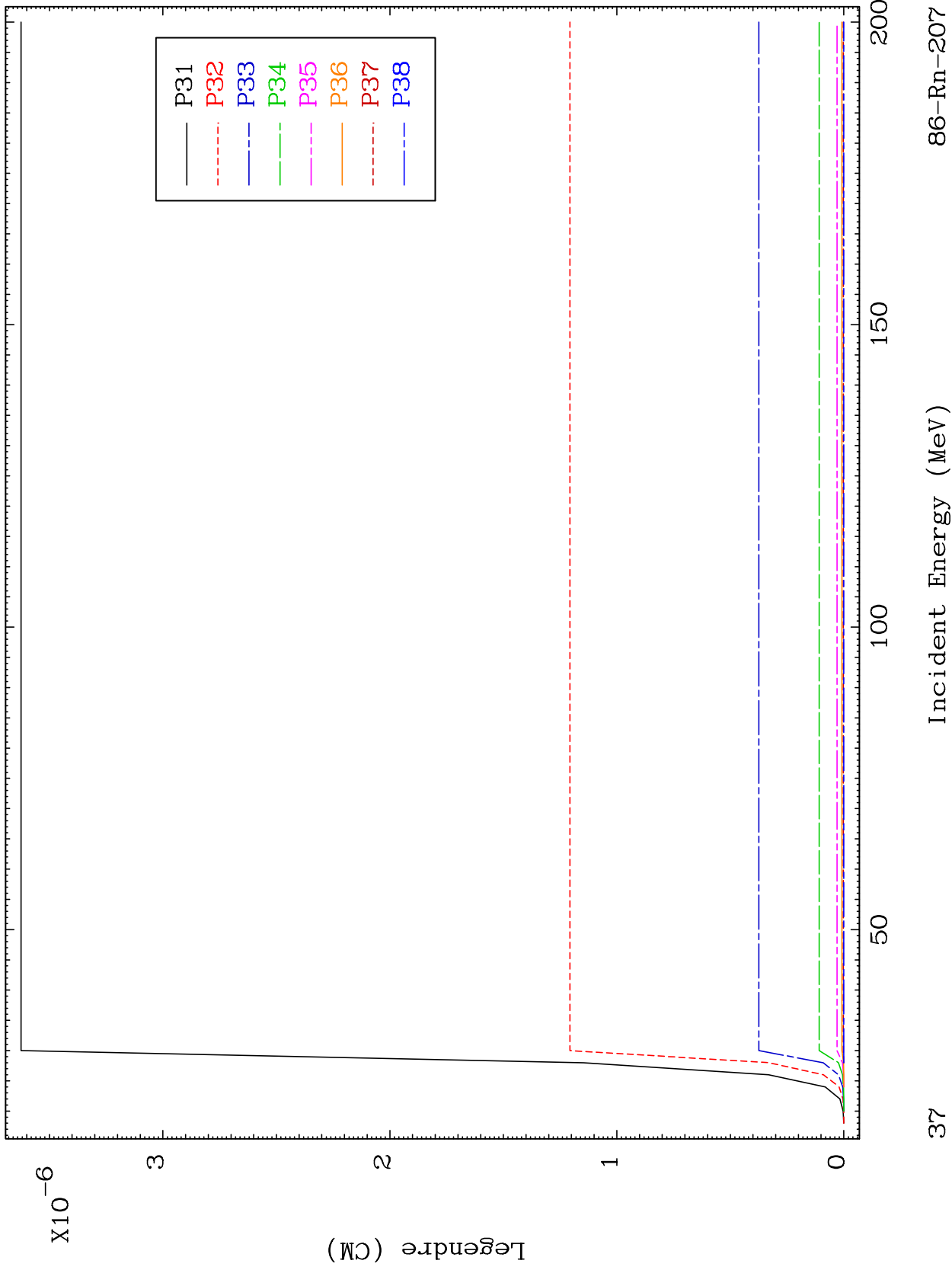




MAT 8613

601.6 keV (n,n') Level
Legendre Coefficients

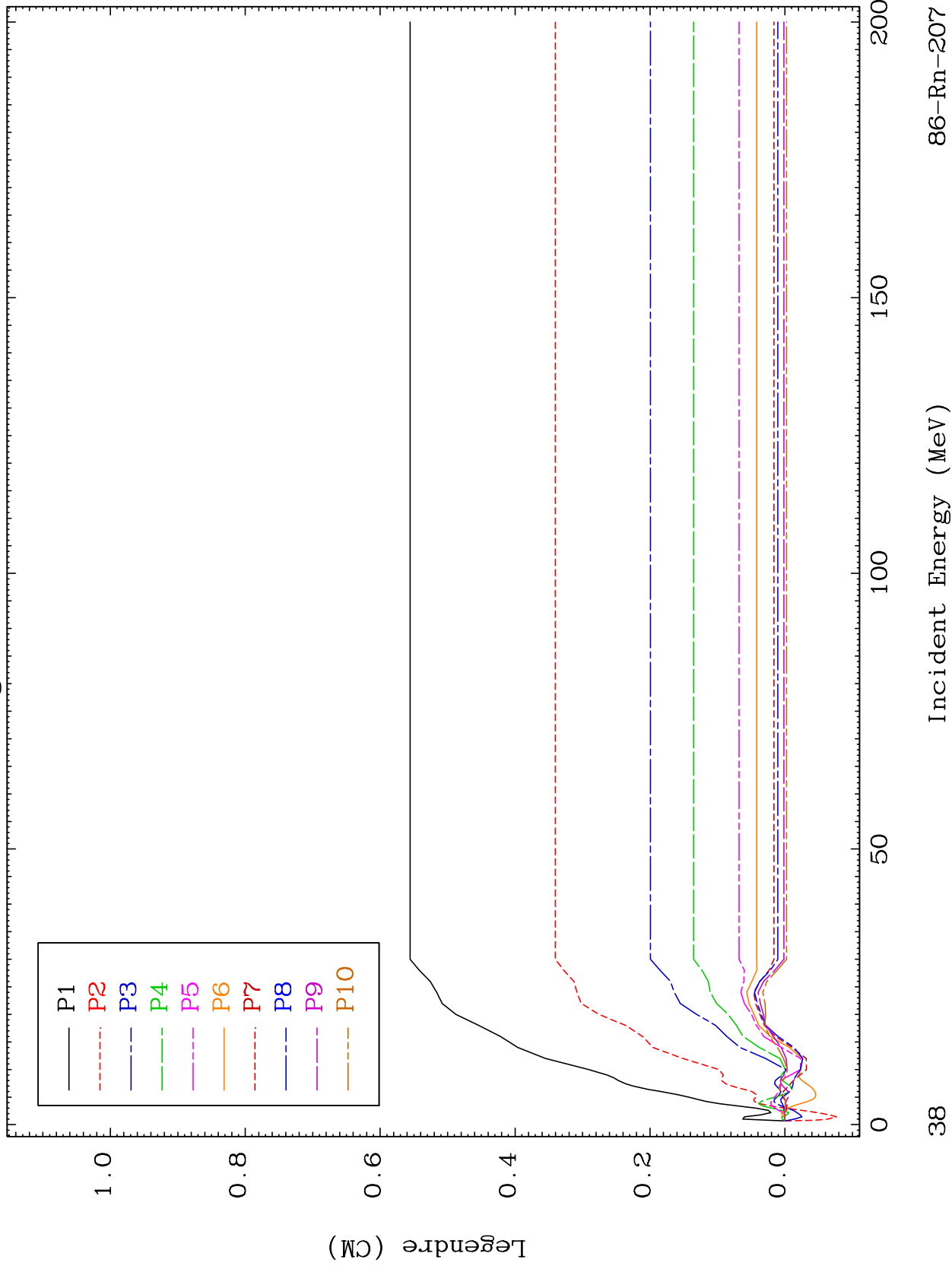
86-Rn-207



MAT 8613

665.1 keV (n,n') Level
Legendre Coefficients

86-Rn-207



86-Rn-207

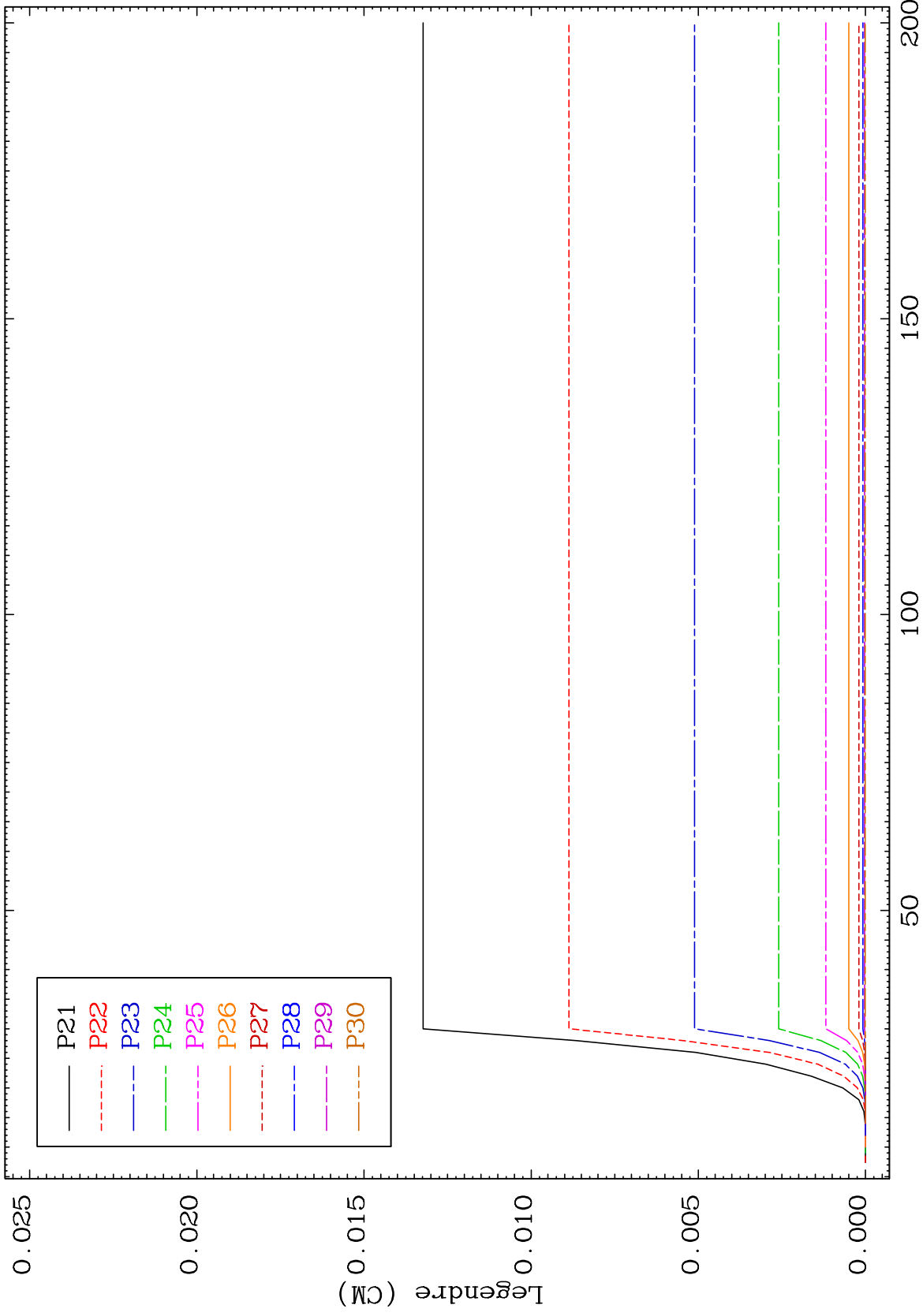
Incident Energy (MeV)

38

MAT 8613

665.1 keV (n, n') Level
Legendre Coefficients

86-Rn-207



40

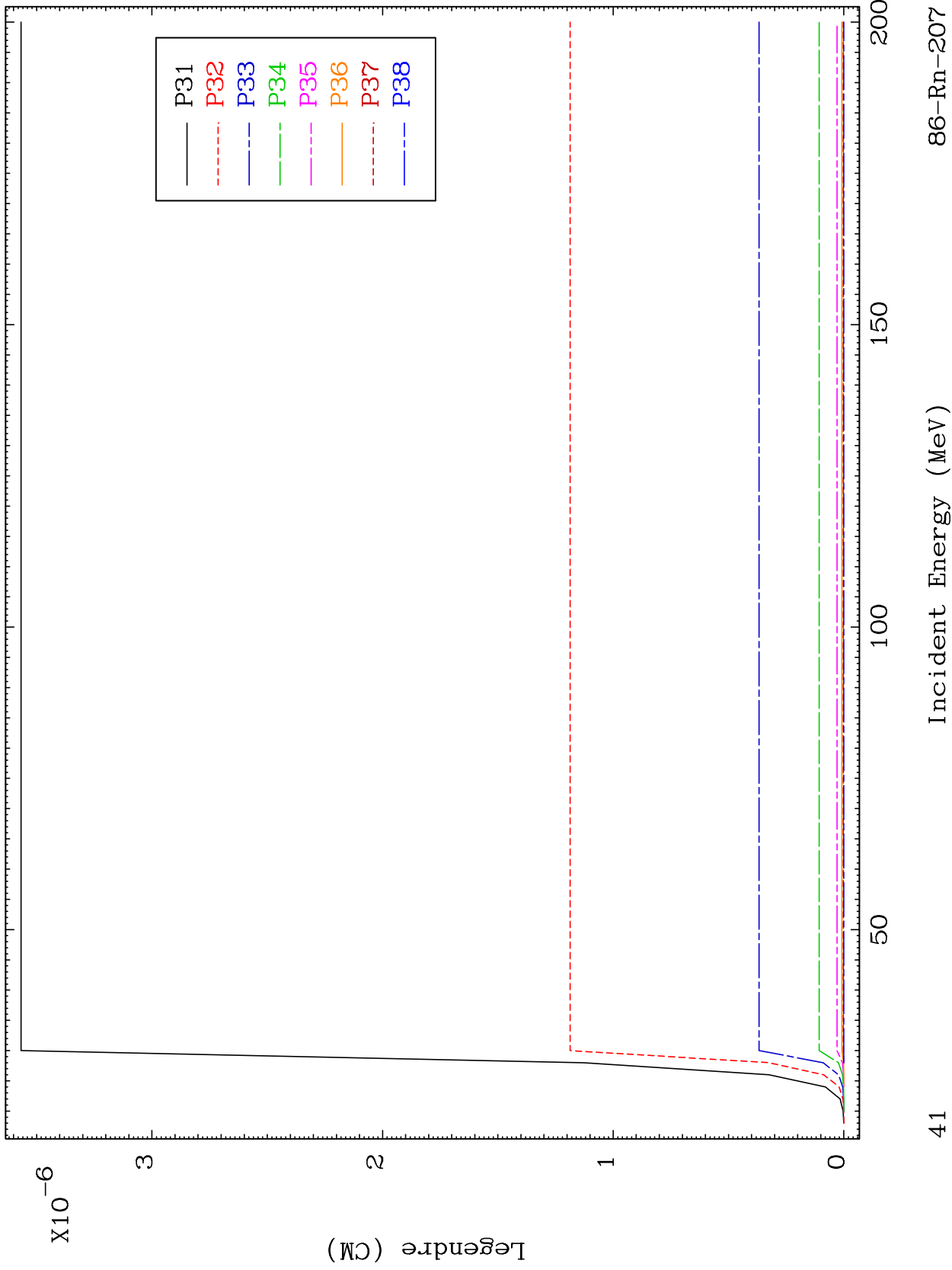
Incident Energy (MeV)

86-Rn-207

MAT 8613

665.1 keV (n,n') Level
Legendre Coefficients

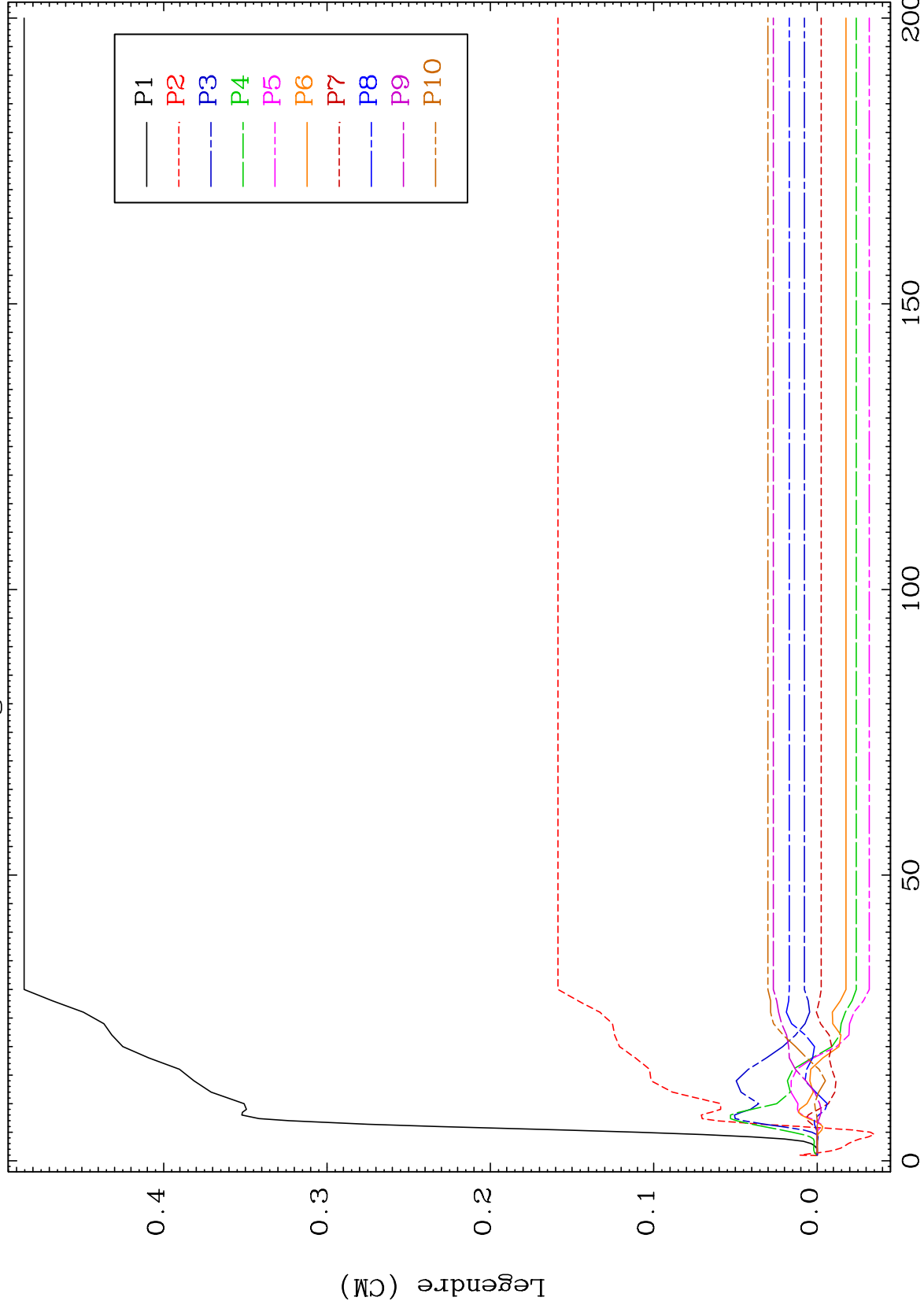
86-Rn-207



MAT 8613

899.1 keV (n,n') Level
Legendre Coefficients

86-Rn-207



86-Rn-207

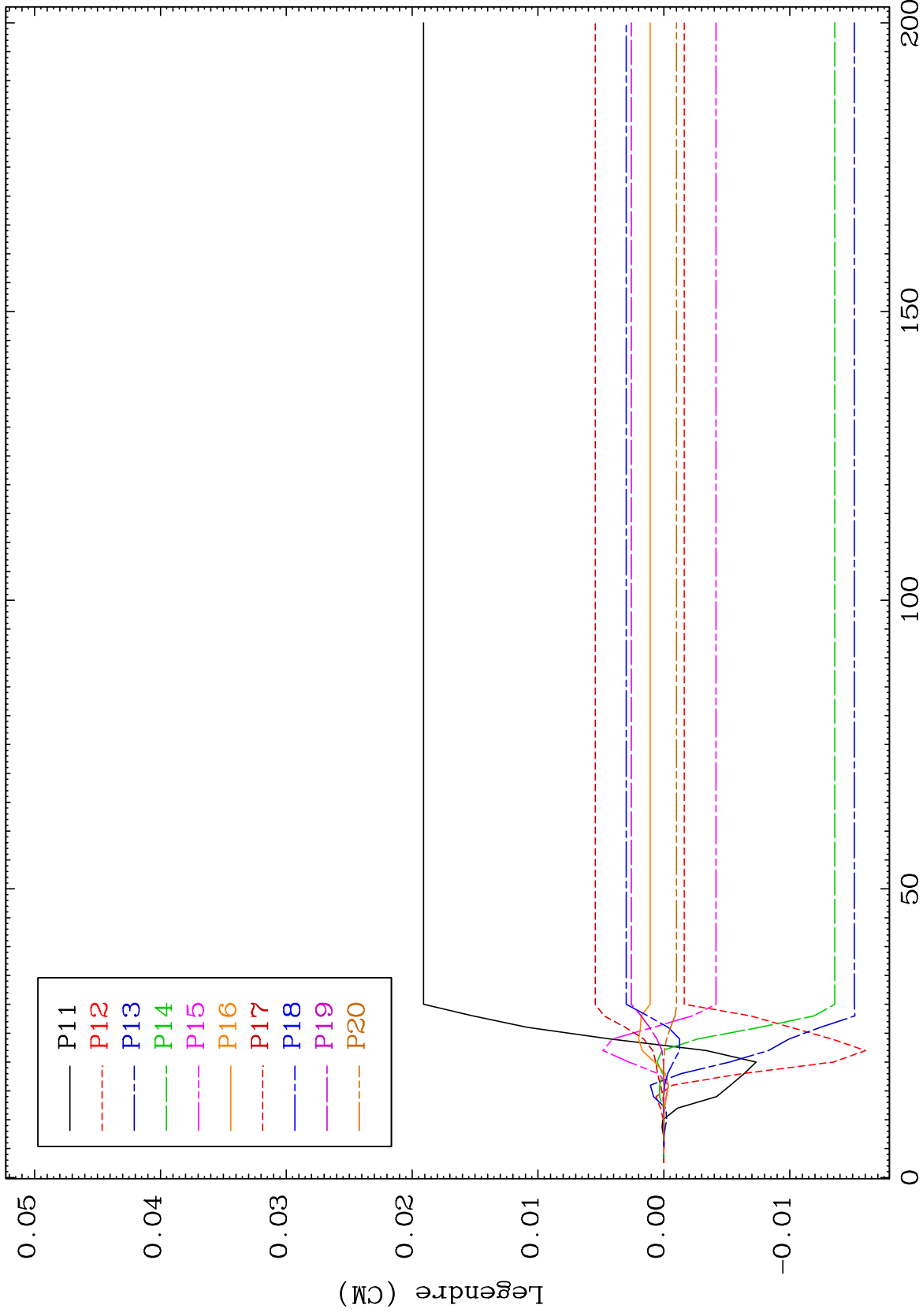
Incident Energy (MeV)

42

MAT 8613

899.1 keV (n,n') Level
Legendre Coefficients

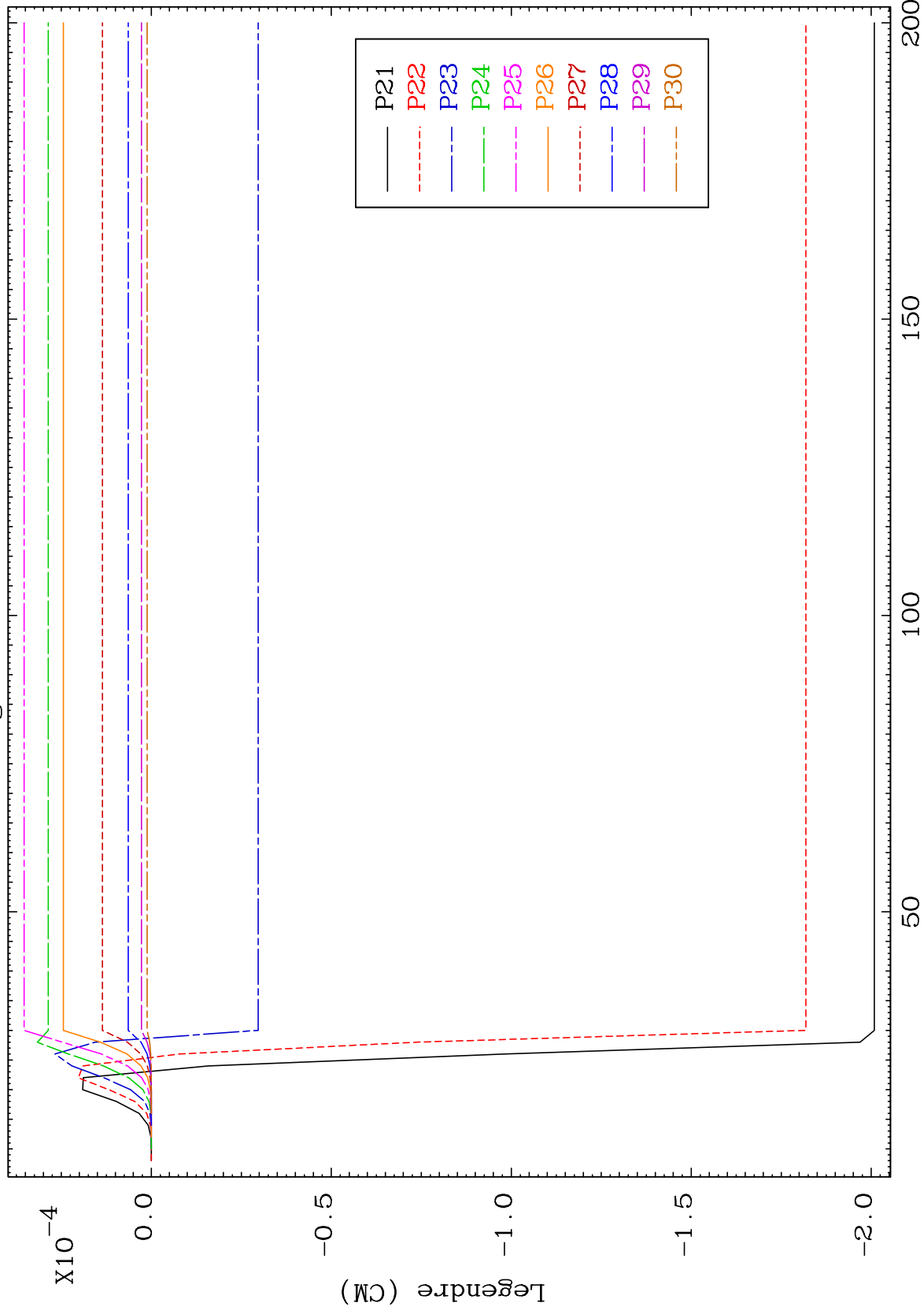
86-Rn-207



43

Incident Energy (MeV)

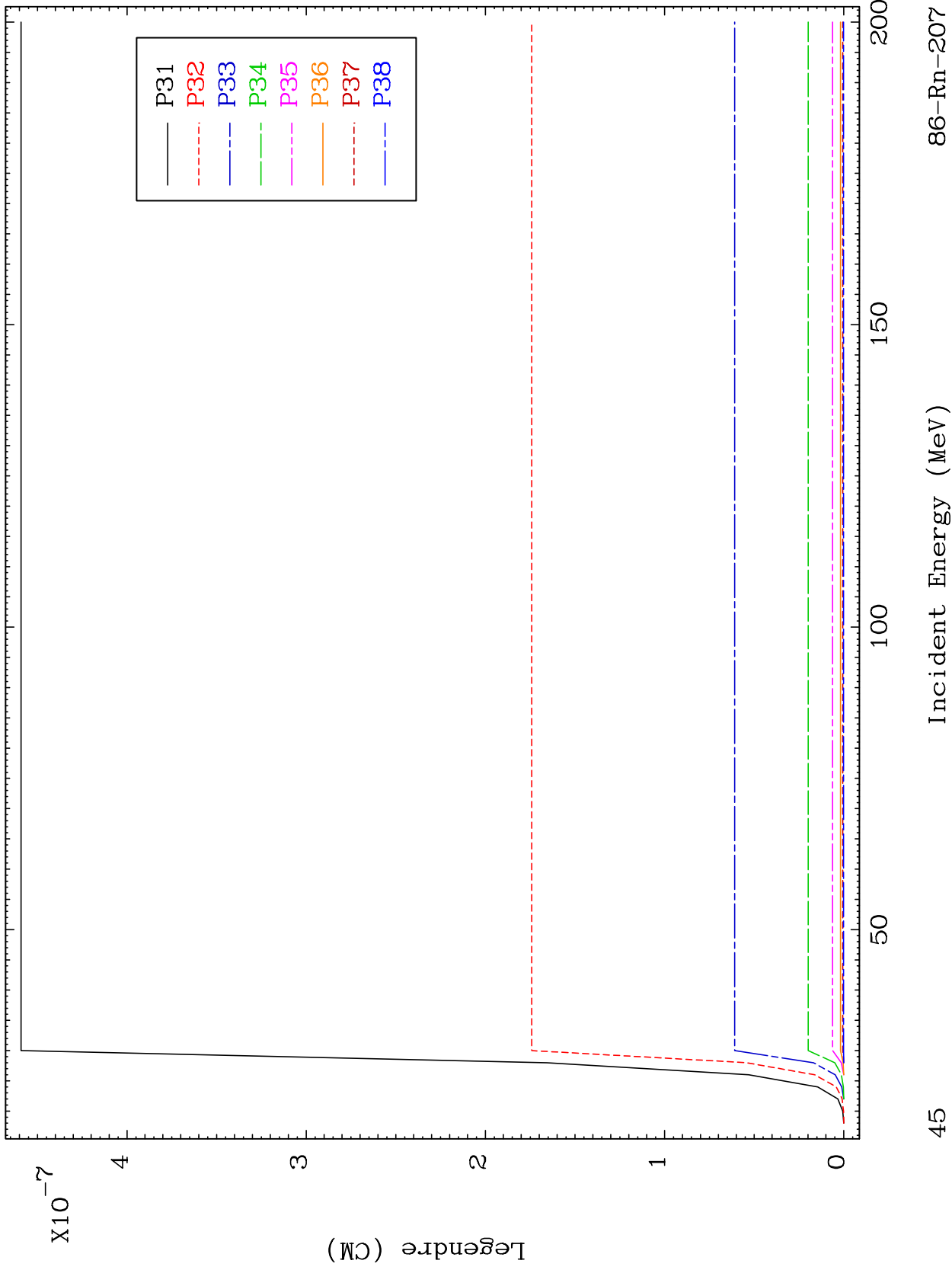
86-Rn-207



MAT 8613

899.1 keV (n,n') Level
Legendre Coefficients

86-Rn-207



86-Rn-207

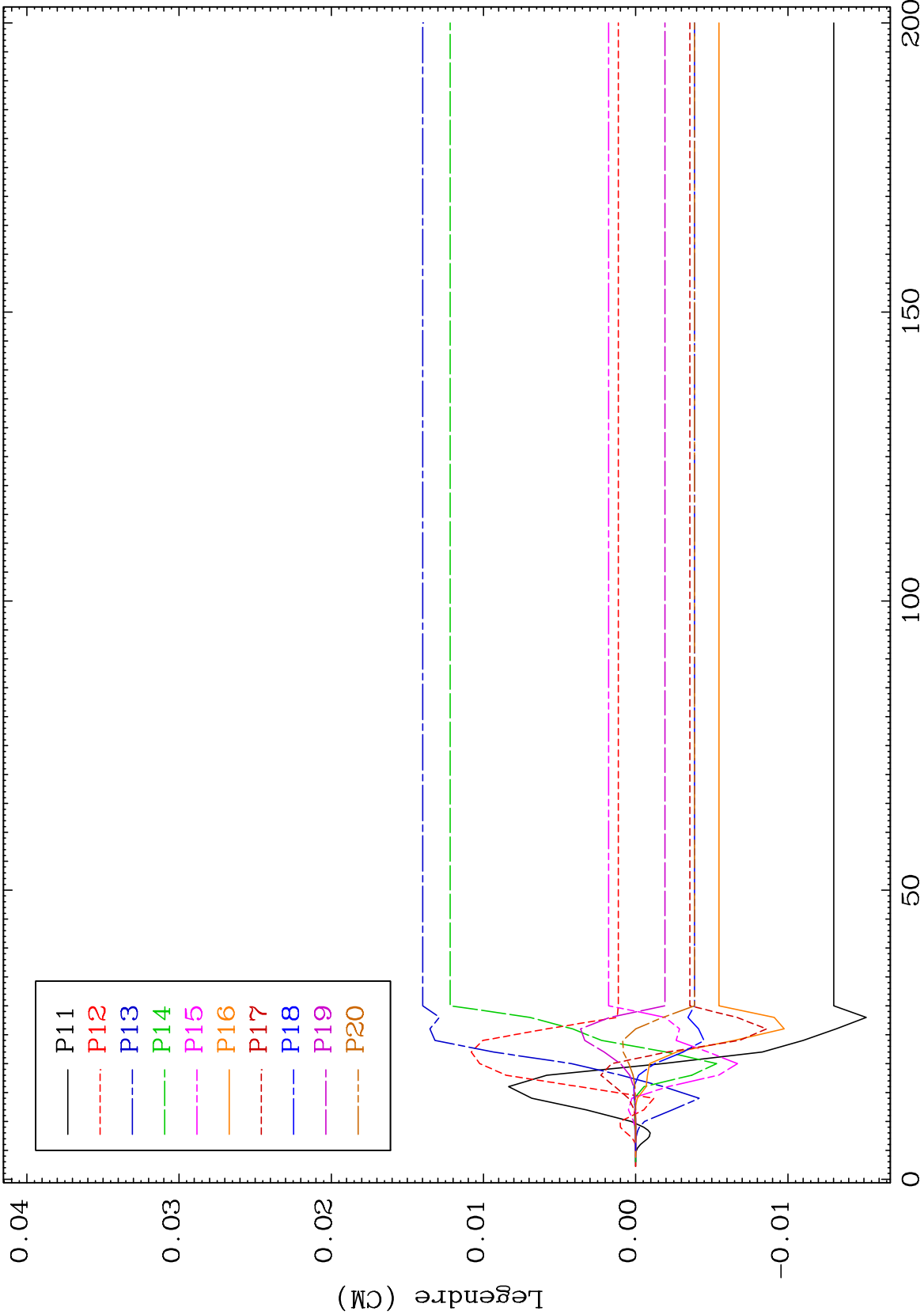
Incident Energy (MeV)

45

MAT 8613

1.397 MeV (n,n') Level Legendre Coefficients

86-Rn-207



47

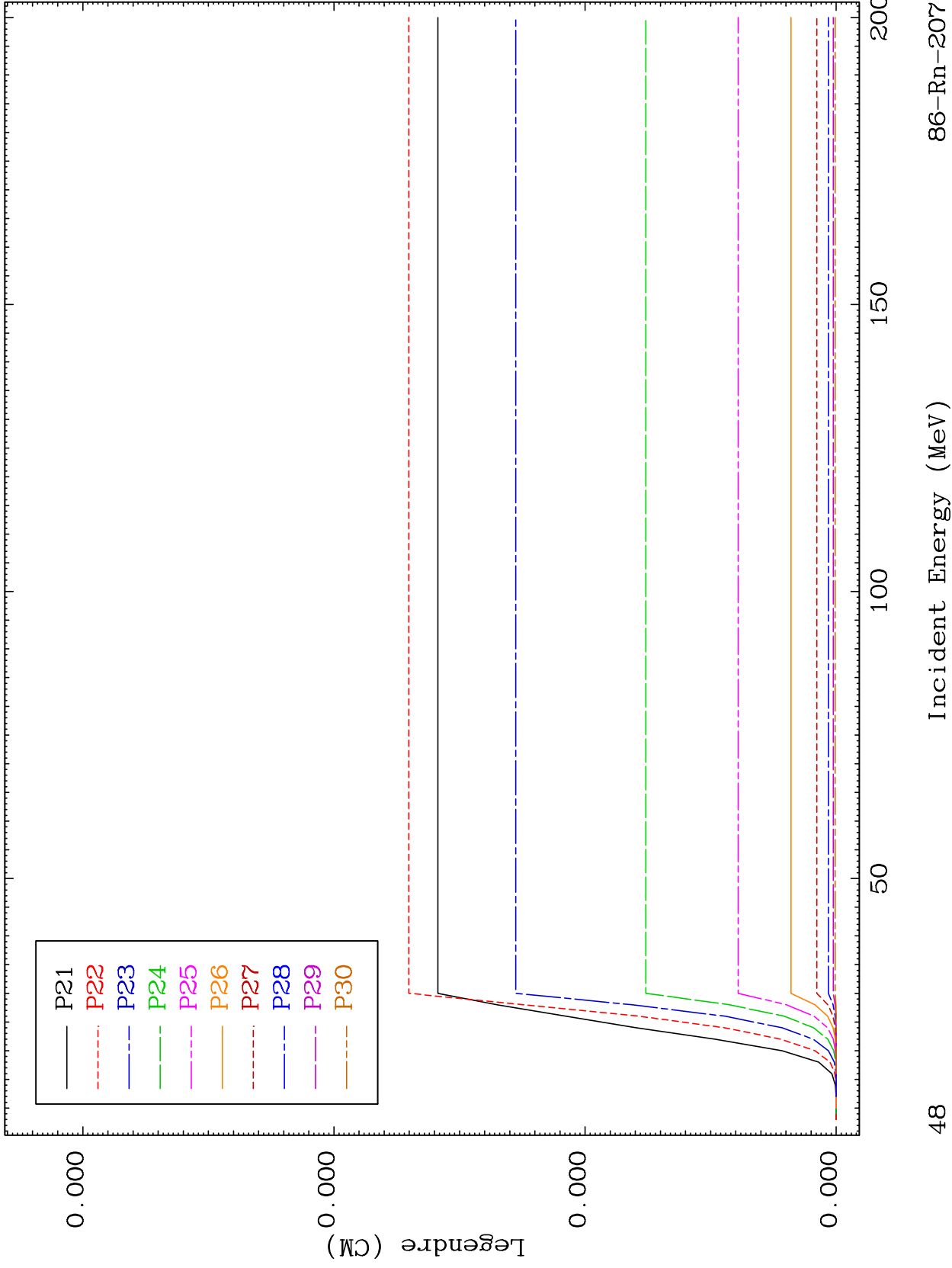
Incident Energy (MeV)

86-Rn-207

MAT 8613

1.397 MeV (n,n') Level Legendre Coefficients

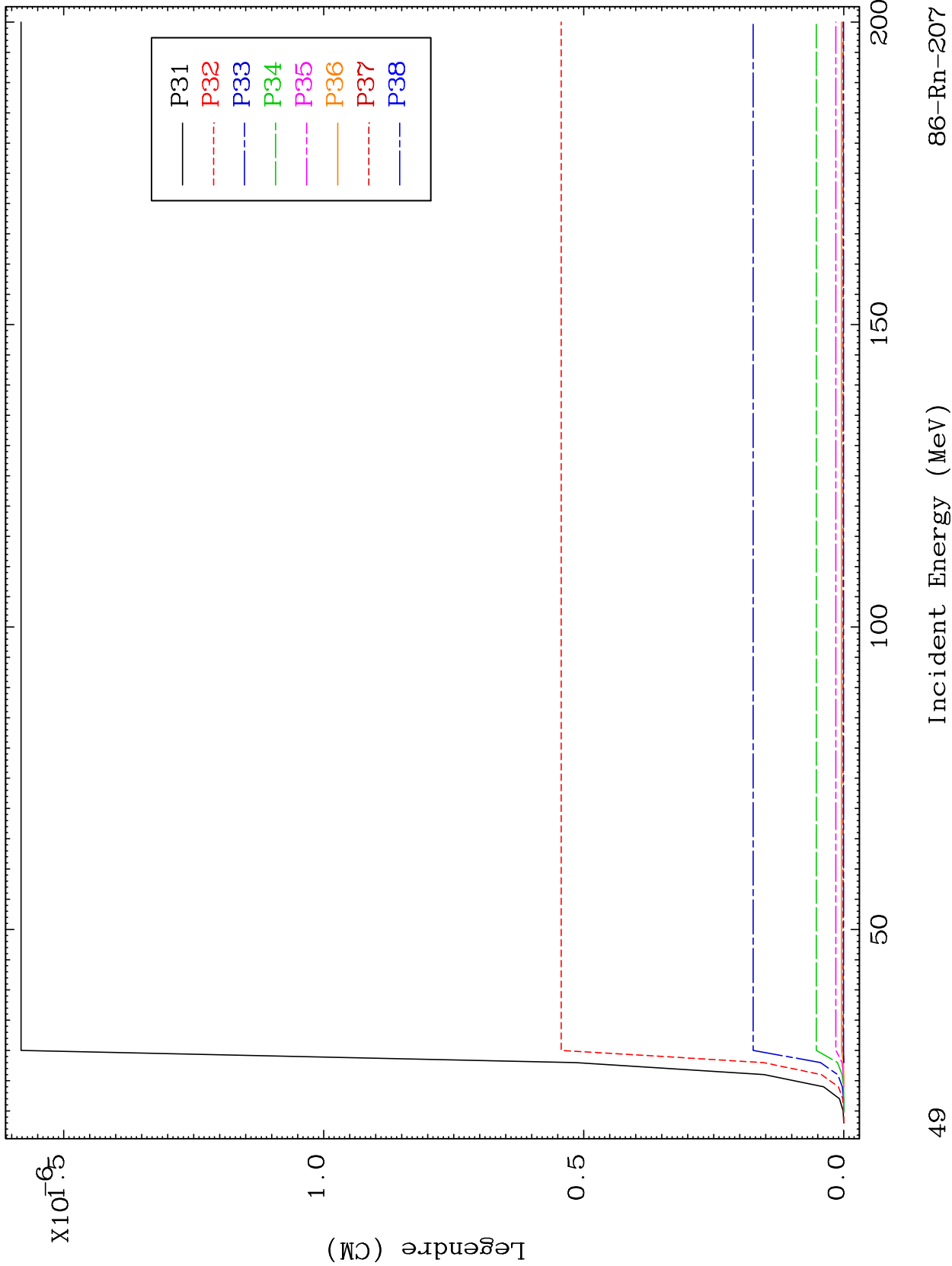
86-Rn-207



48

Incident Energy (MeV)

86-Rn-207

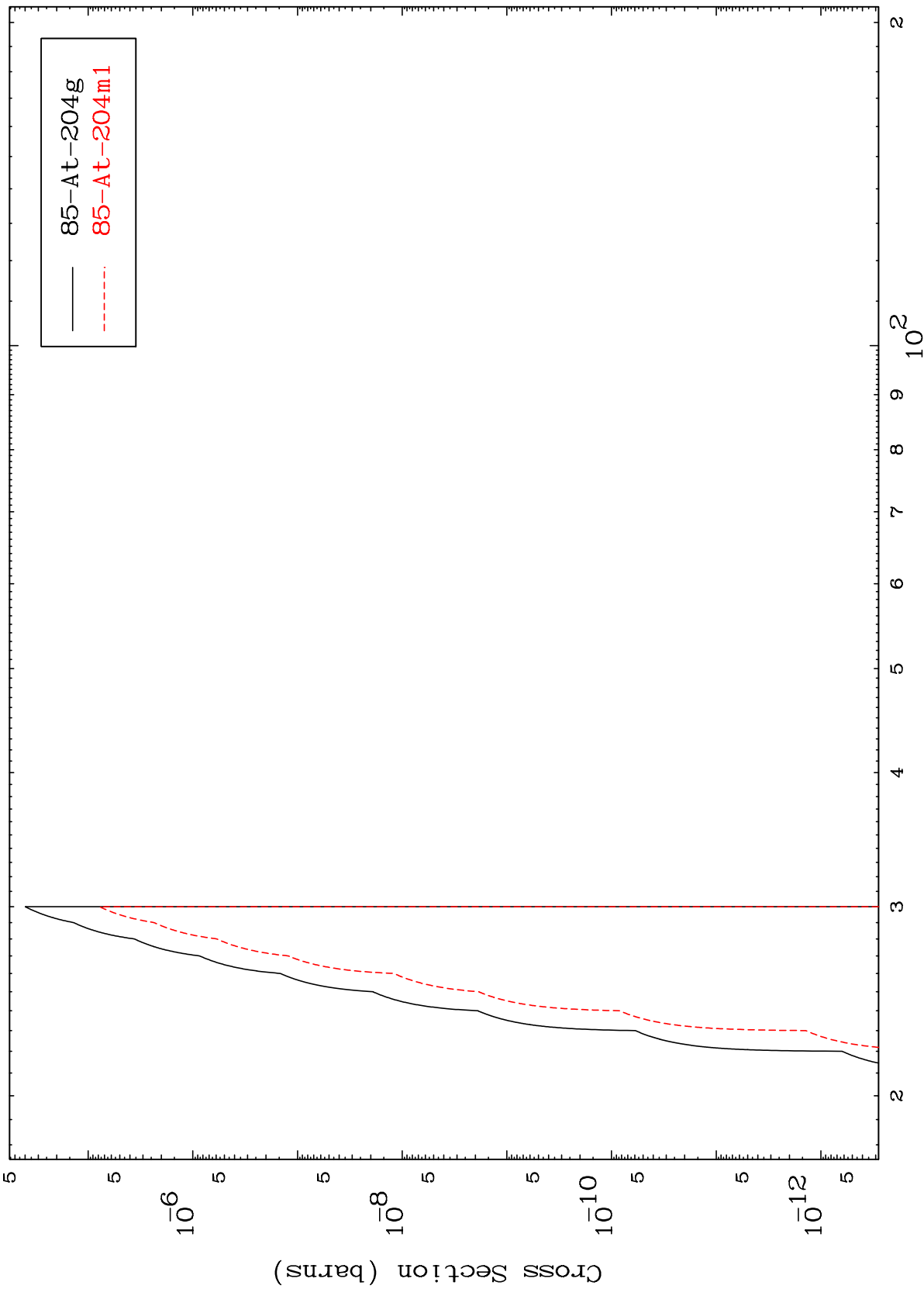


MAT 8613

(n,2n) d

86-Rn-207

Radionuclide Production Cross Section



85-At-204g
85-At-204m1

50

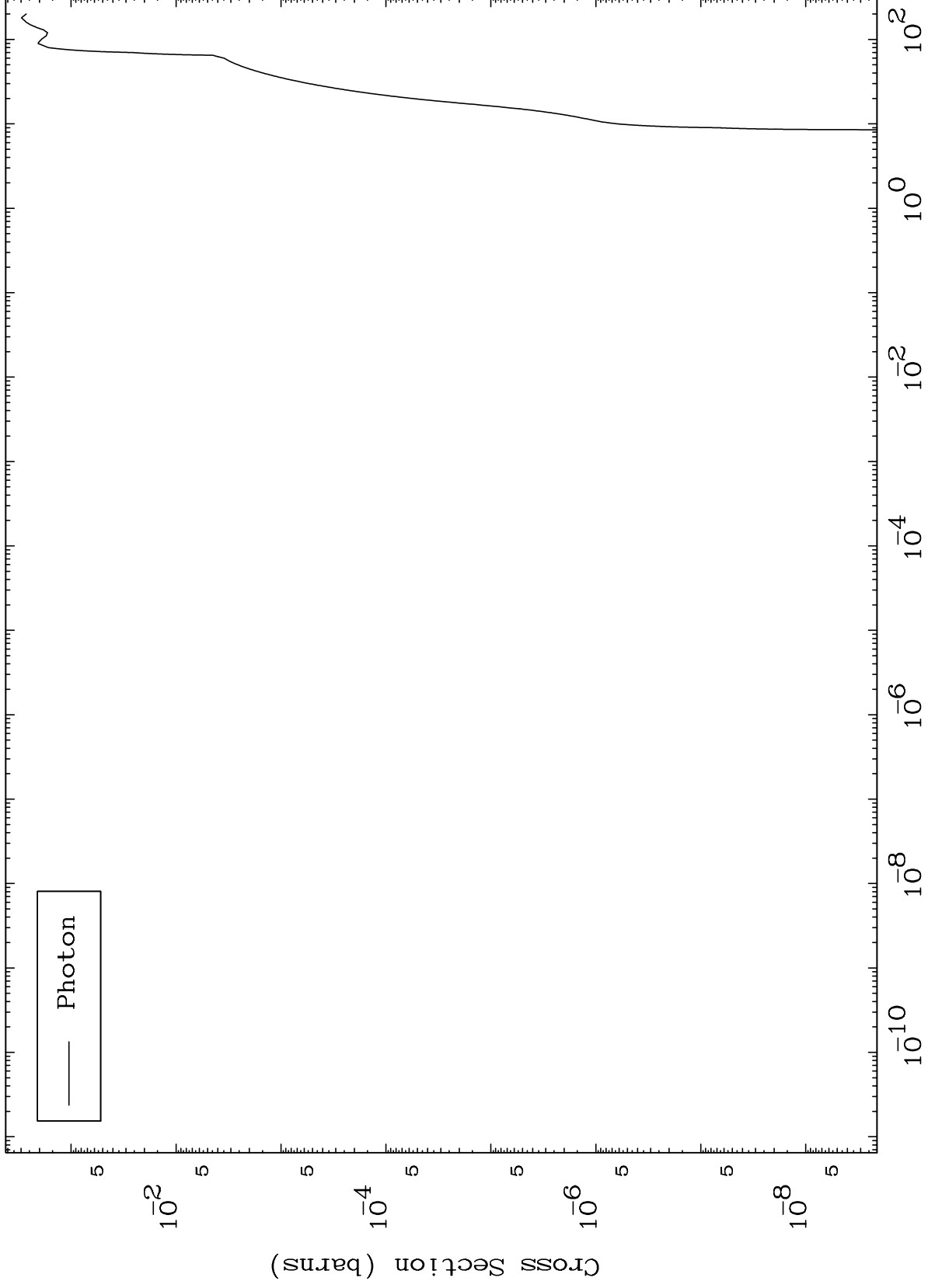
Incident Energy (MeV)

86-Rn-207

MAT 8613

Fission
Radionuclide Production Cross Section

86-Rn-207



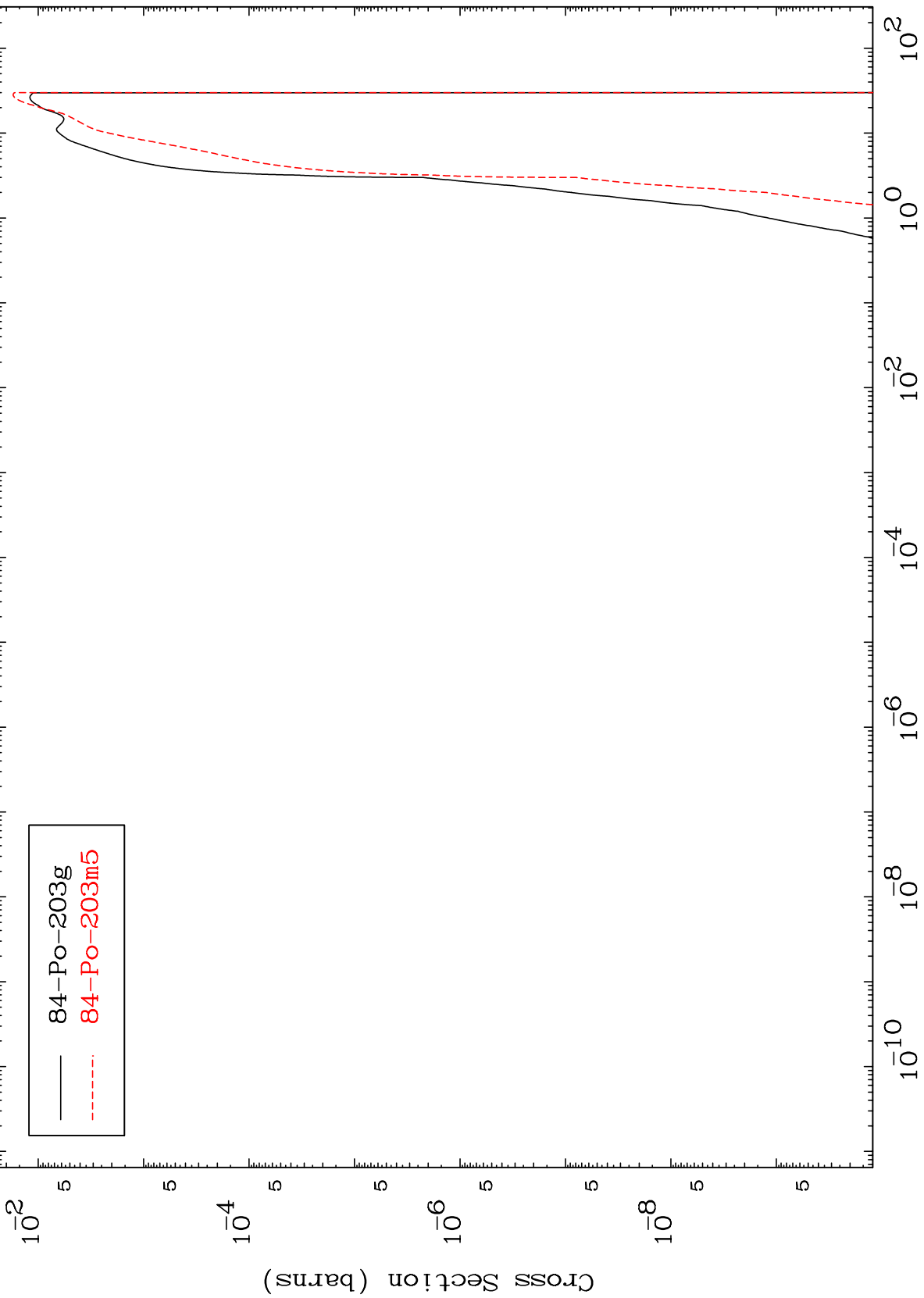
86-Rn-207

MAT 8613

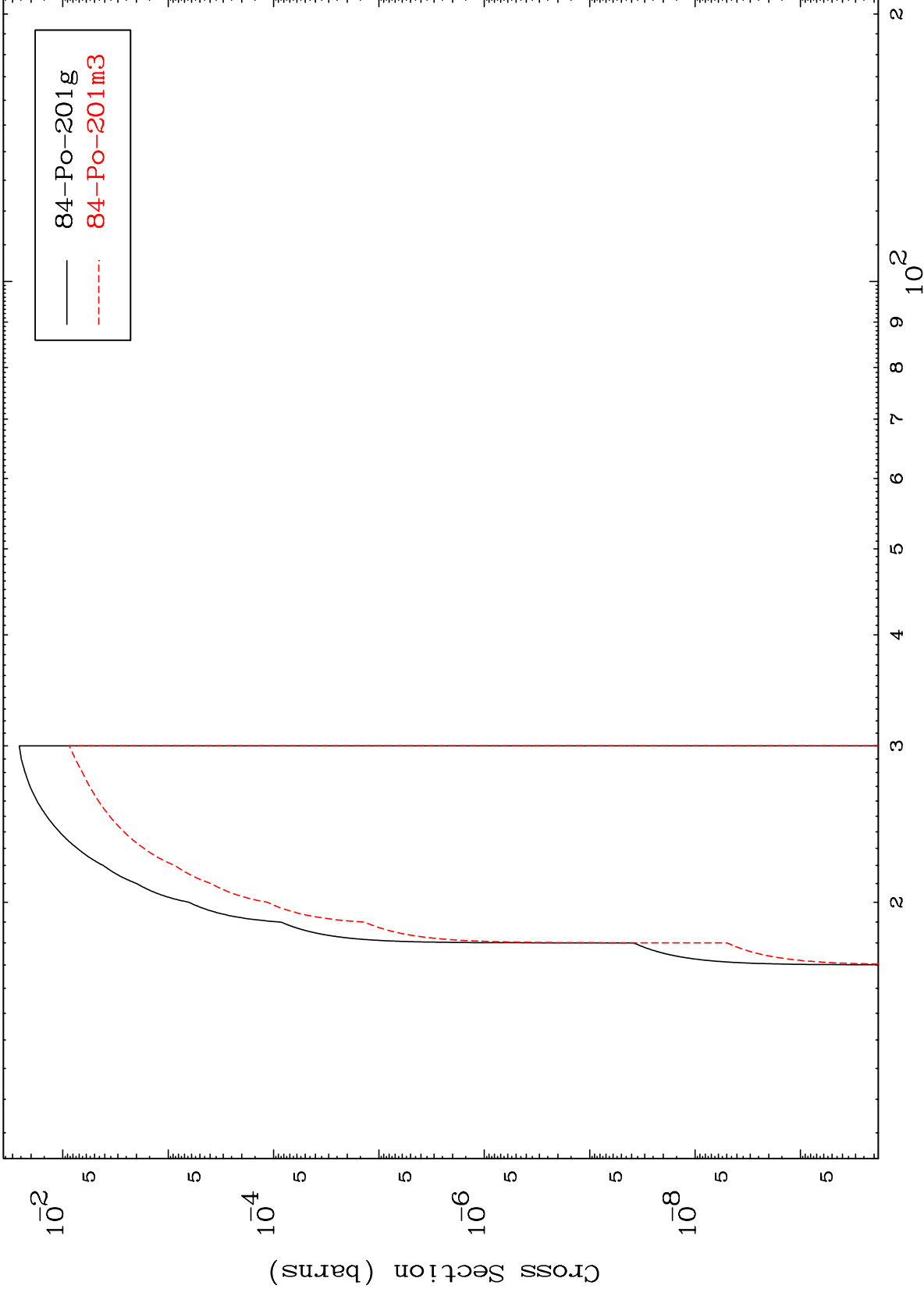
(n, n') α

86-Rn-207

Radionuclide Production Cross Section



Radionuclide Production Cross Section

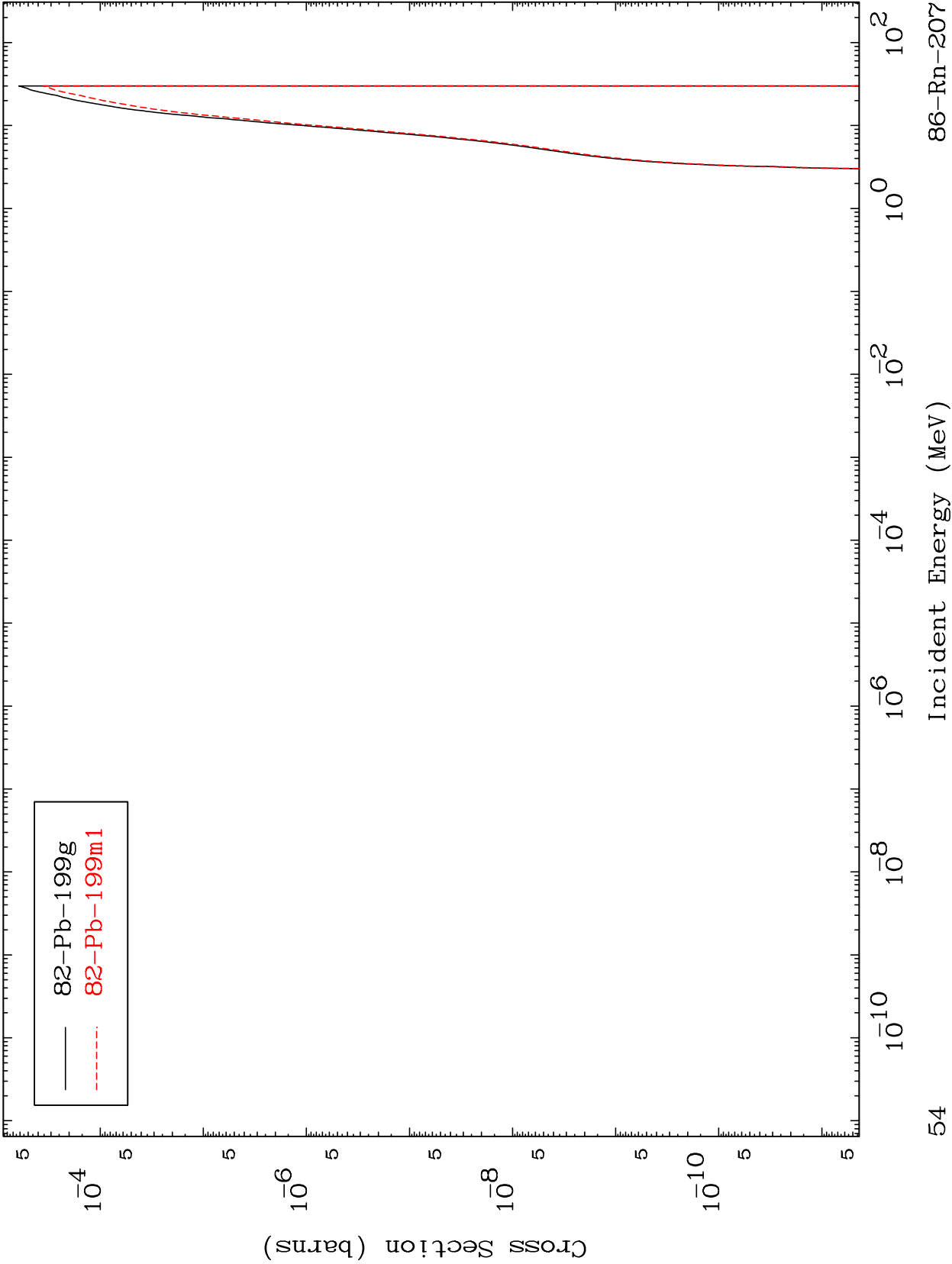


MAT 8613

(n,n') 2α

86-Rn-207

Radionuclide Production Cross Section



54

Incident Energy (MeV)

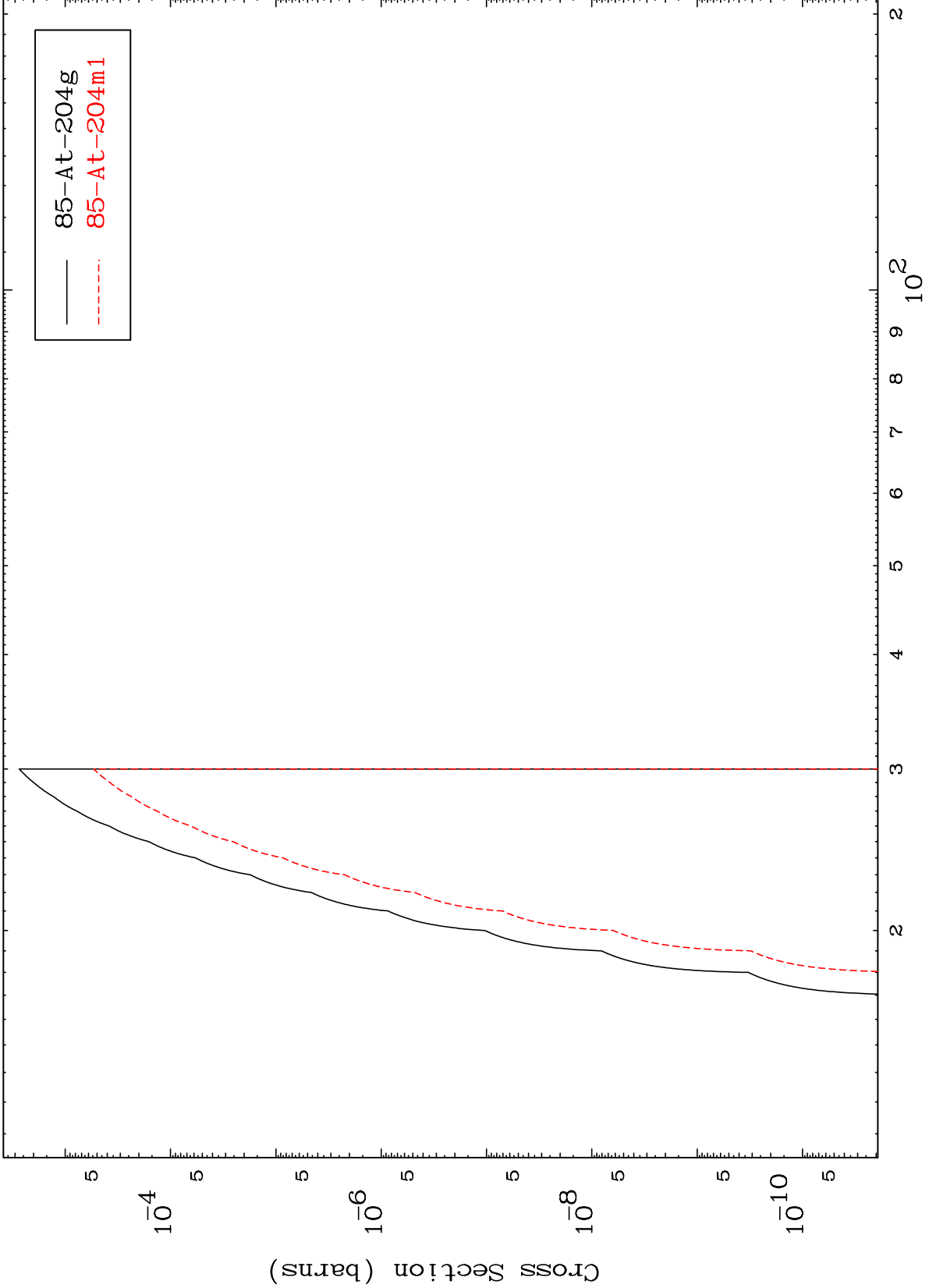
86-Rn-207

MAT 8613

(n,n') t

86-Rn-207

Radionuclide Production Cross Section



55

Incident Energy (MeV)

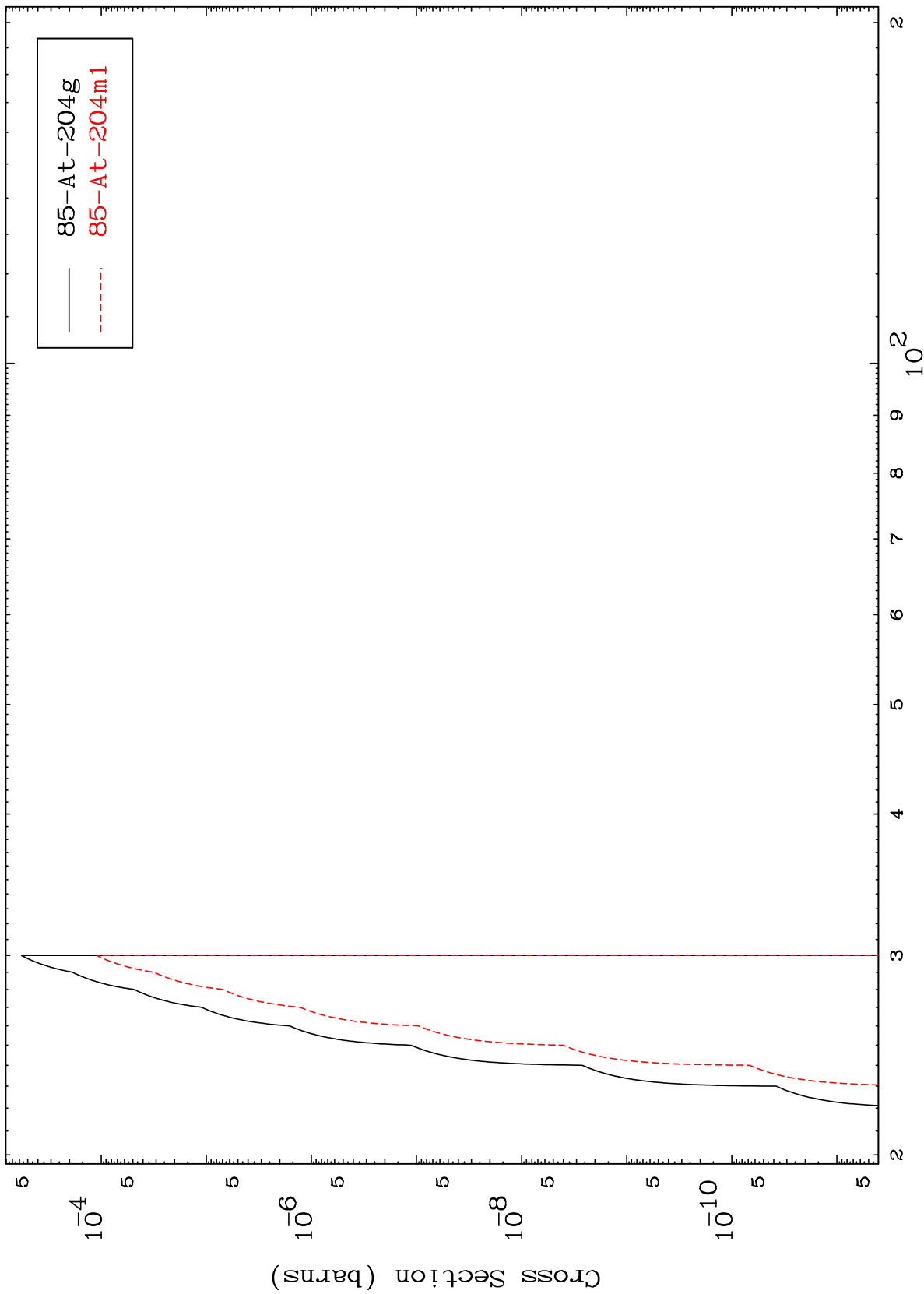
86-Rn-207

MAT 8613

(n,3n) p

86-Rn-207

Radionuclide Production Cross Section



56

Incident Energy (MeV)

86-Rn-207

MAT 8613

(n,p) α

86-Rn-207

Radionuclide Production Cross Section

