

Program EVALPLOT
(Version 2015-2)

by

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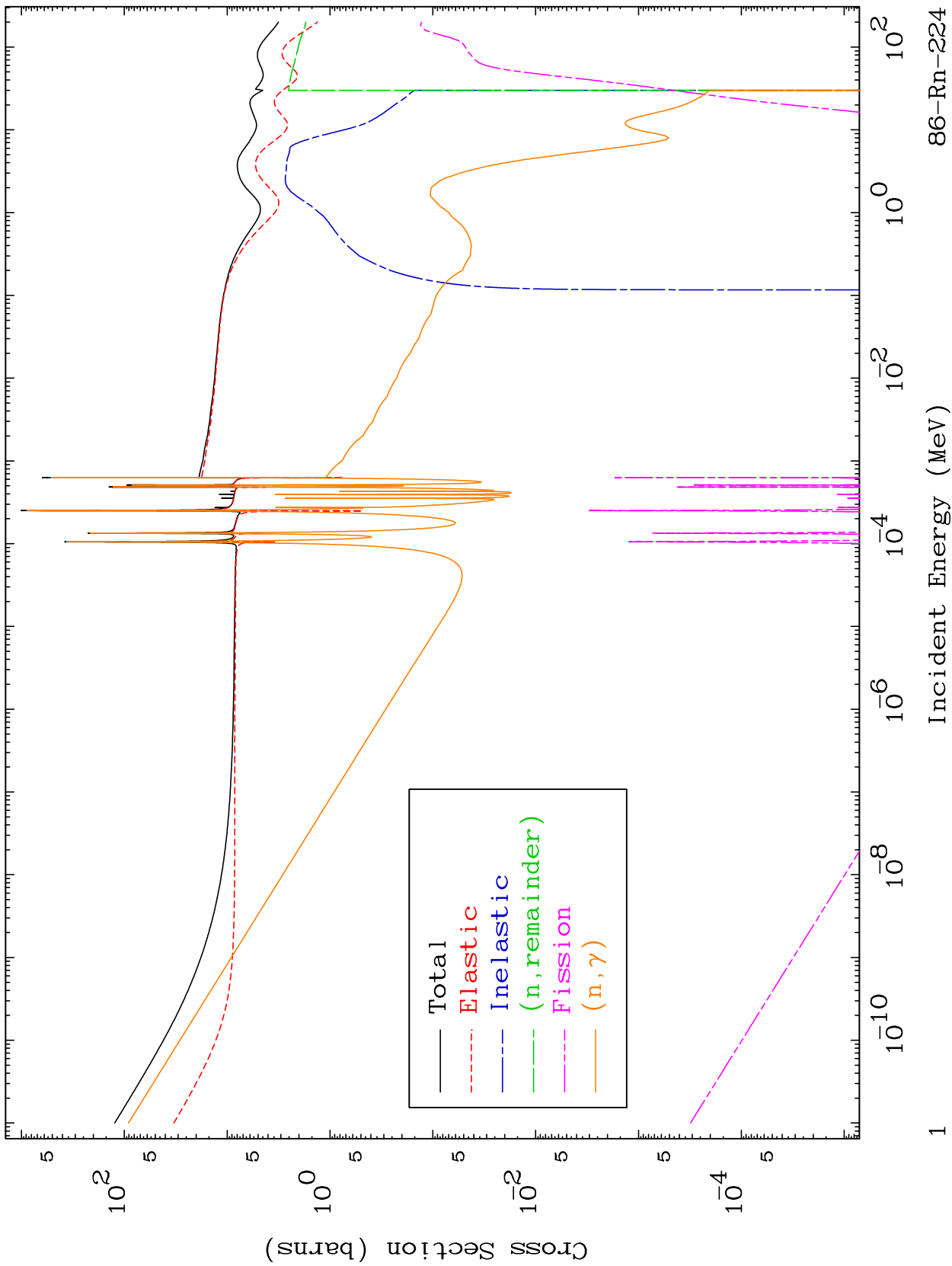
Press Mouse Button to Start

MAT 8664

Major

293 Kelvin Cross Sections

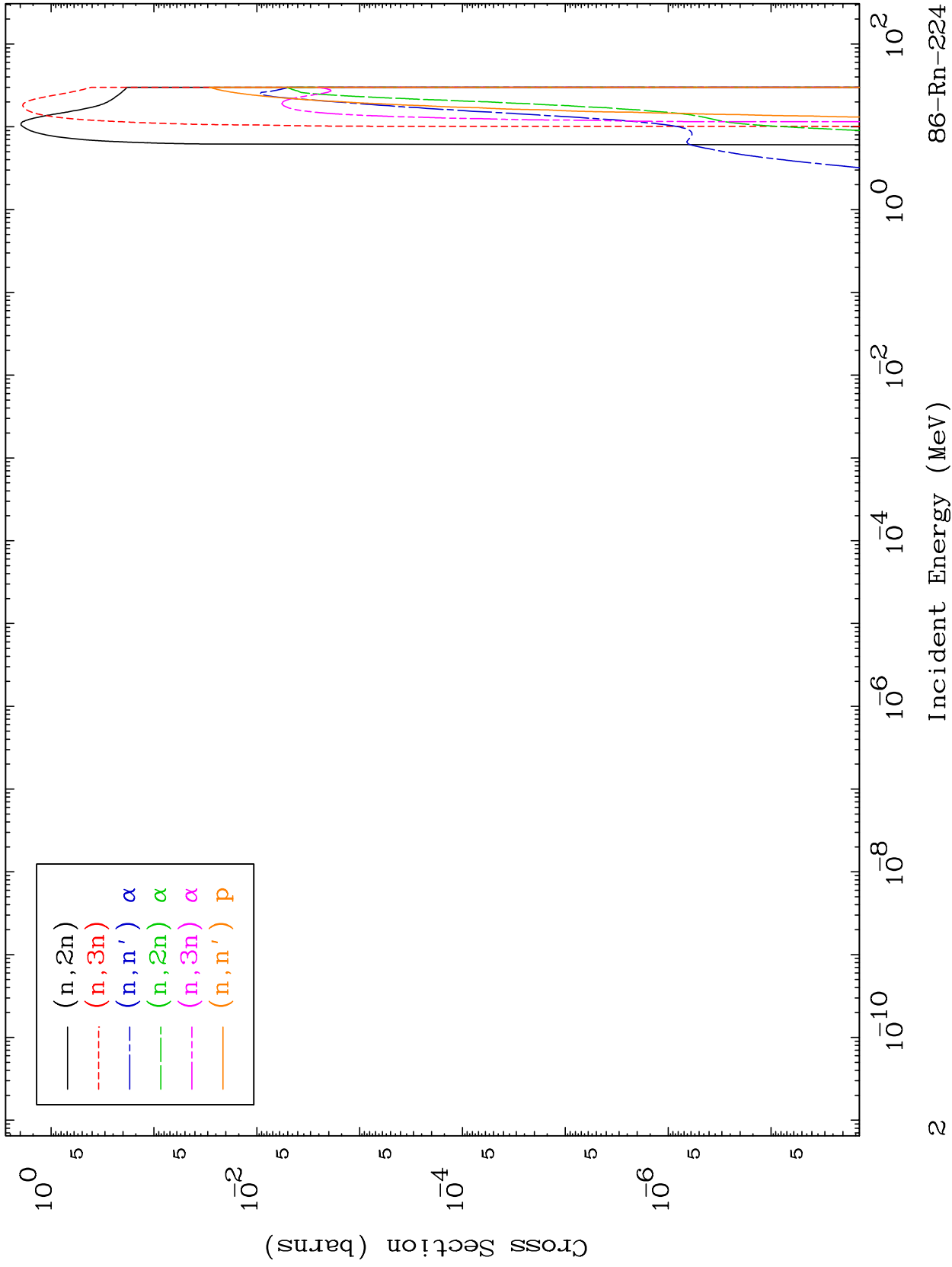
86-Rn-224



MAT 8664

Neutron Production
293 Kelvin Cross Sections

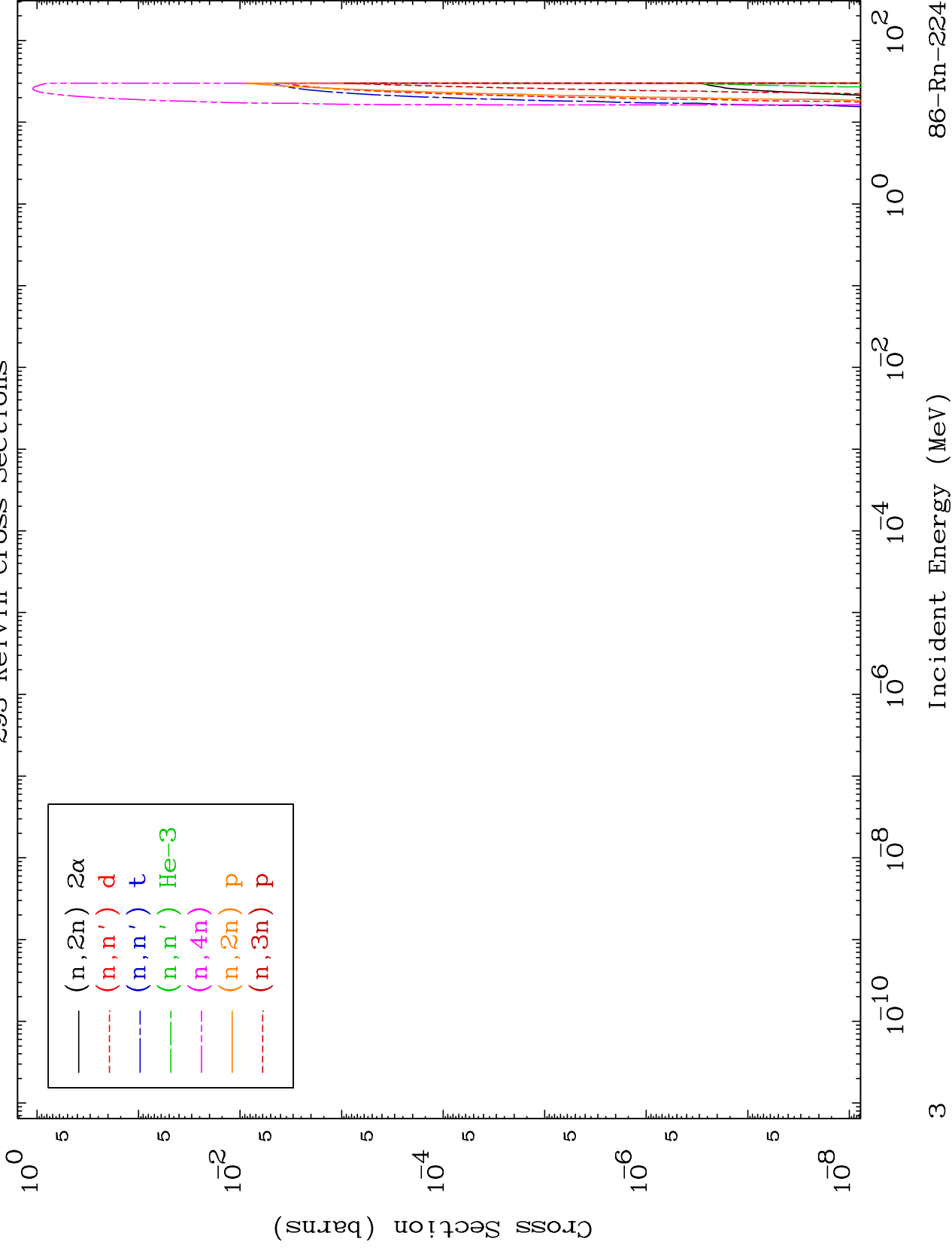
86-Rn-224



MAT 8664

Neutron Production
293 Kelvin Cross Sections

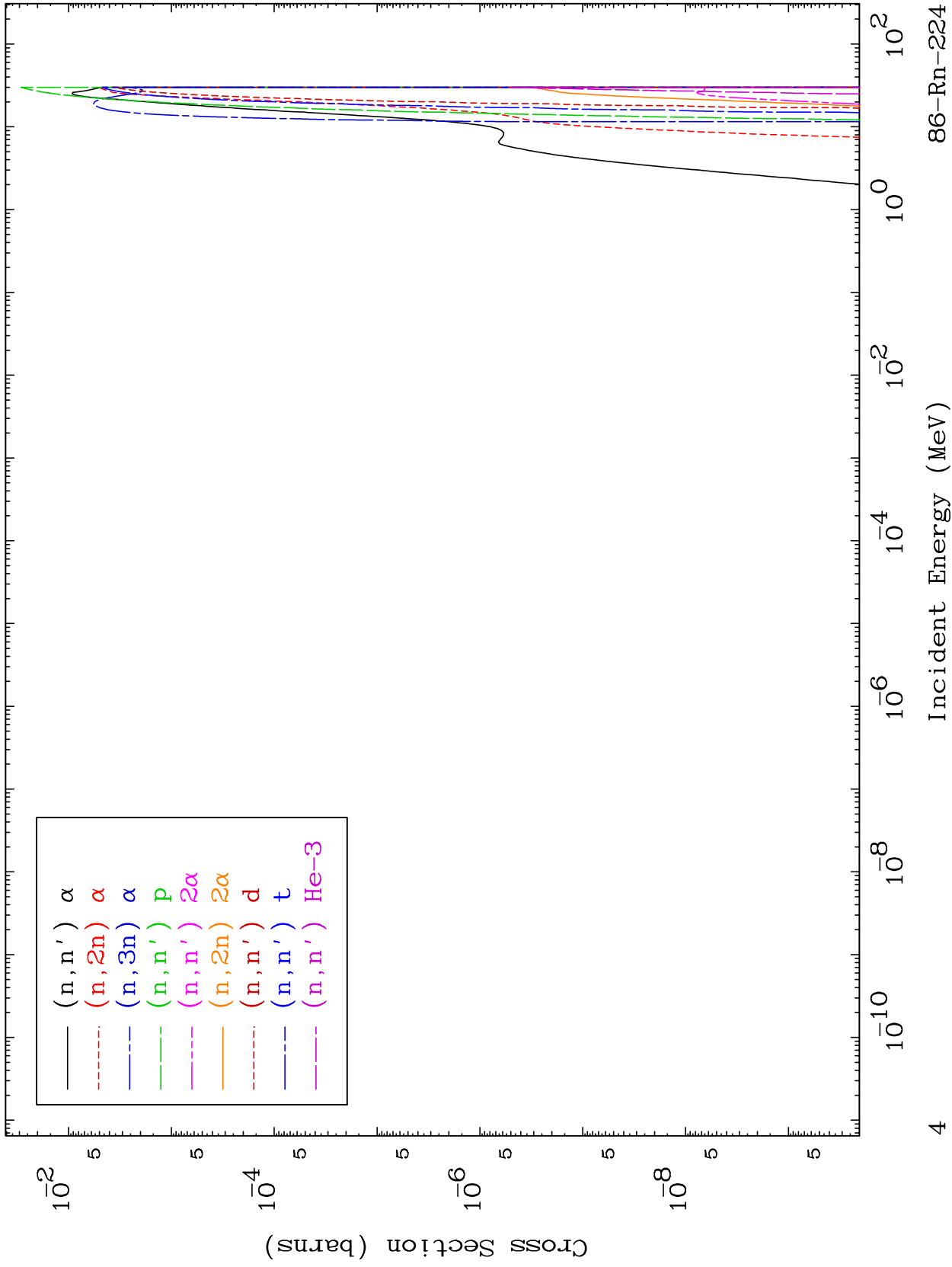
86-Rn-224



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Charged Particle
293 Kelvin Cross Sections

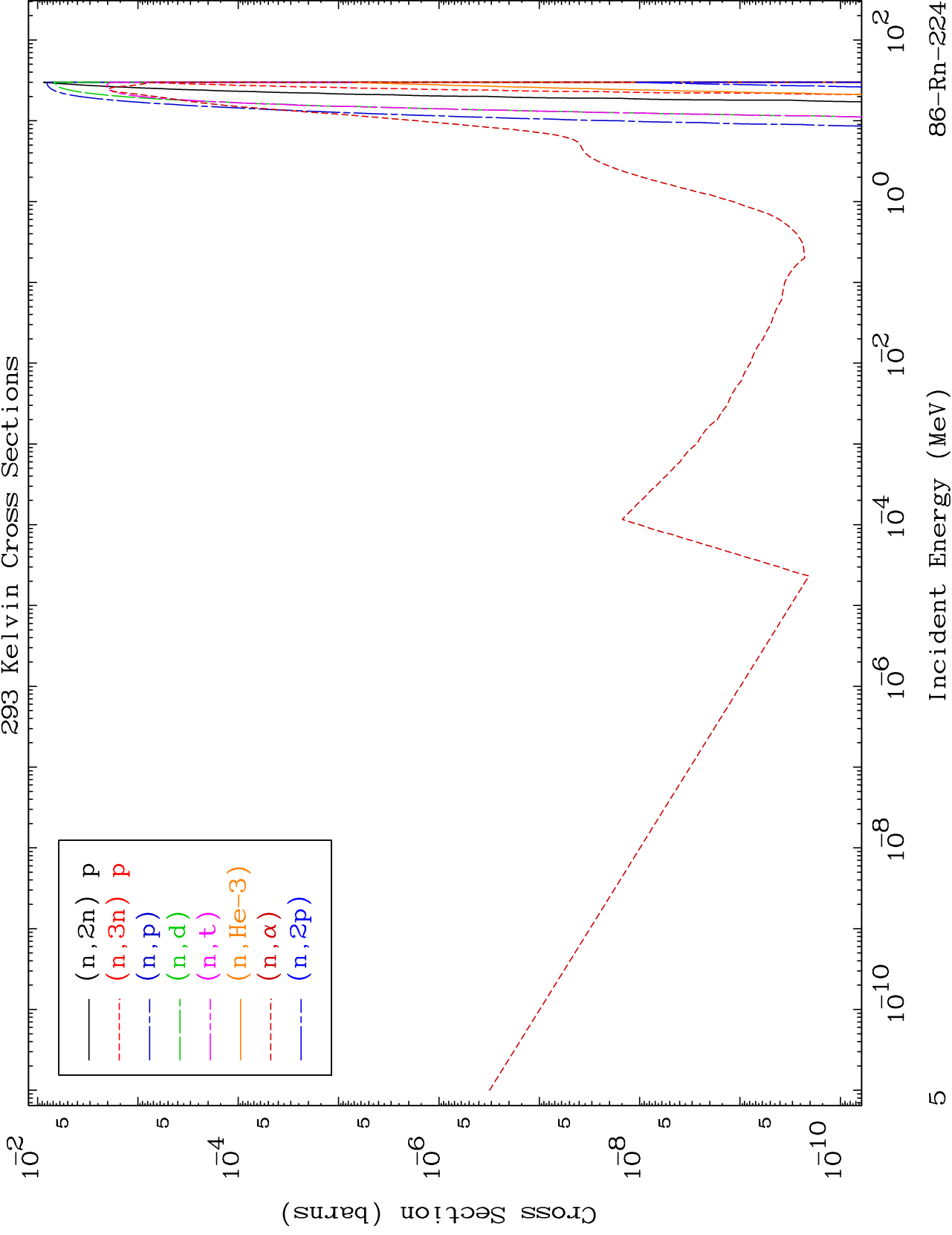
86-Rn-224



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Charged Particle
293 Kelvin Cross Sections

86-Rn-224



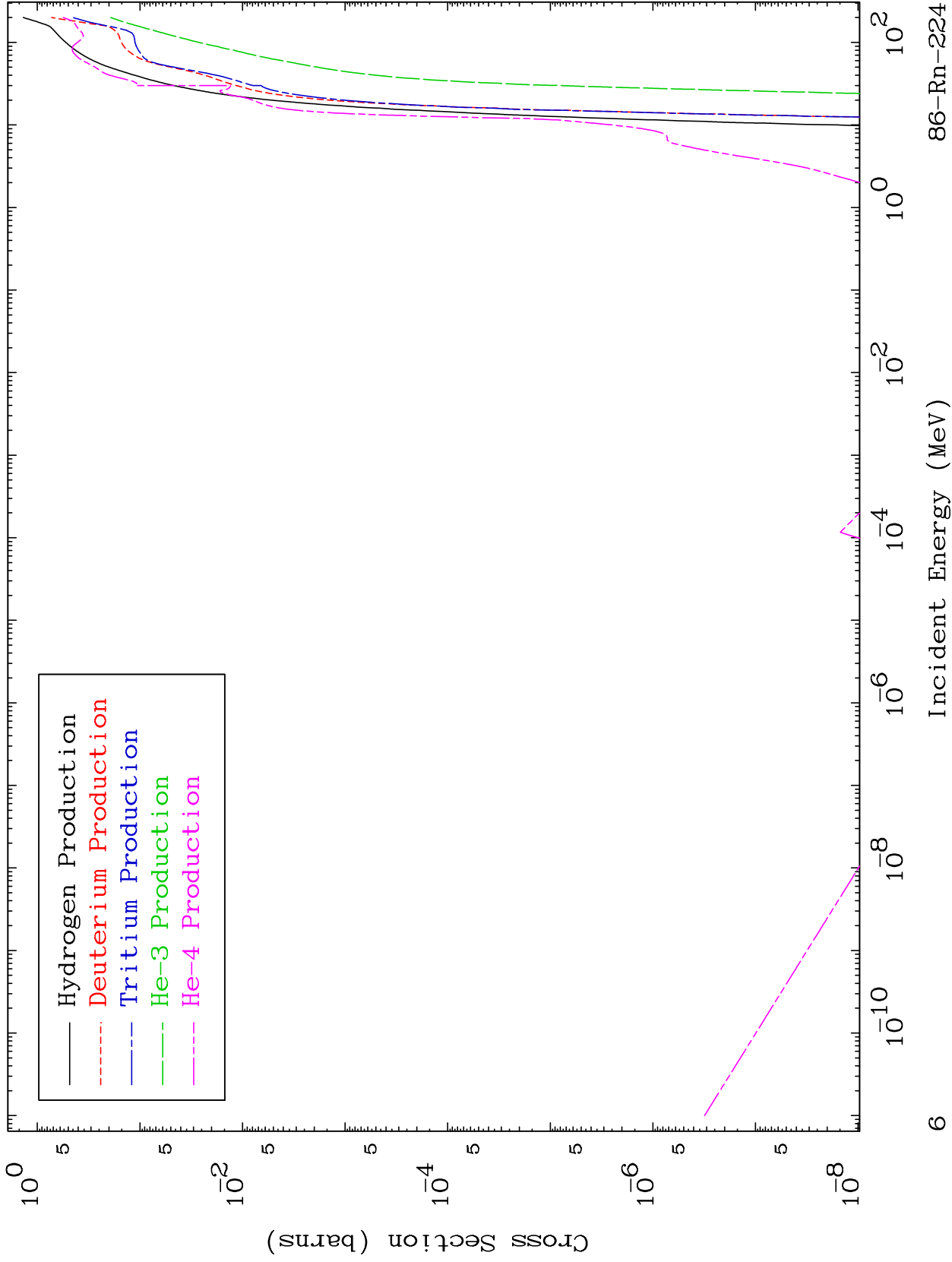
5

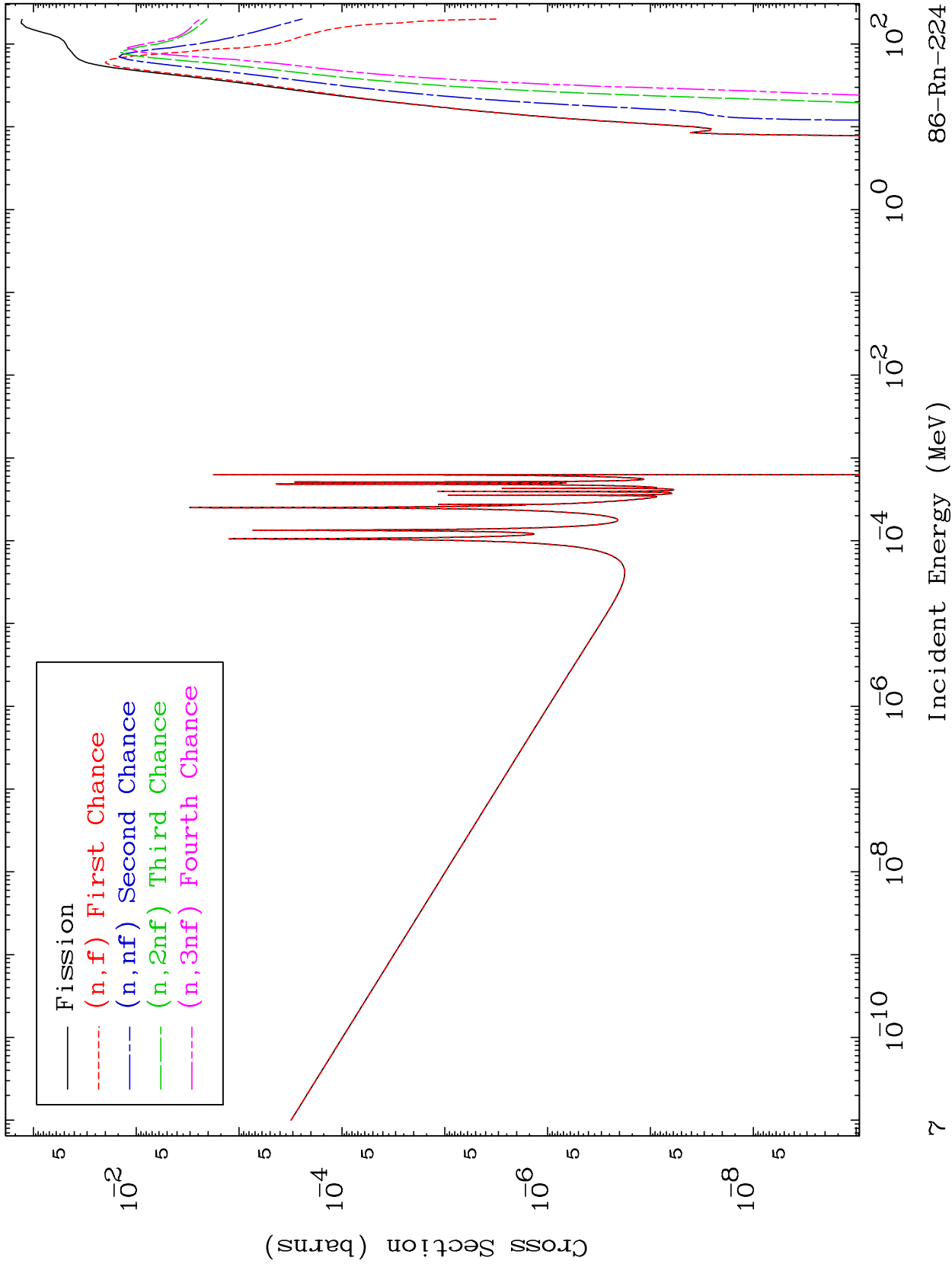
86-Rn-224

MAT 8664

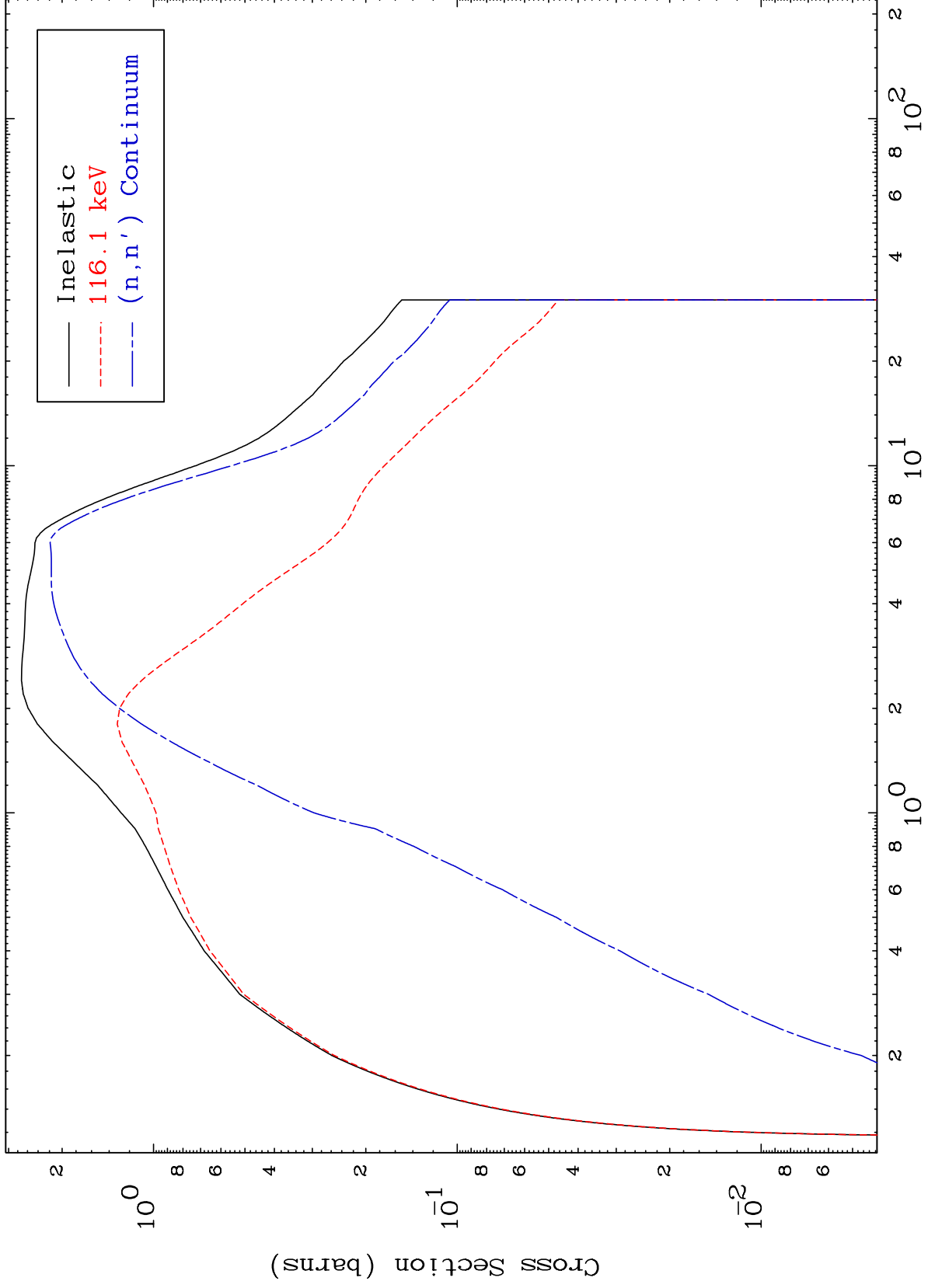
Particle Production
293 Kelvin Cross Sections

86-Rn-224





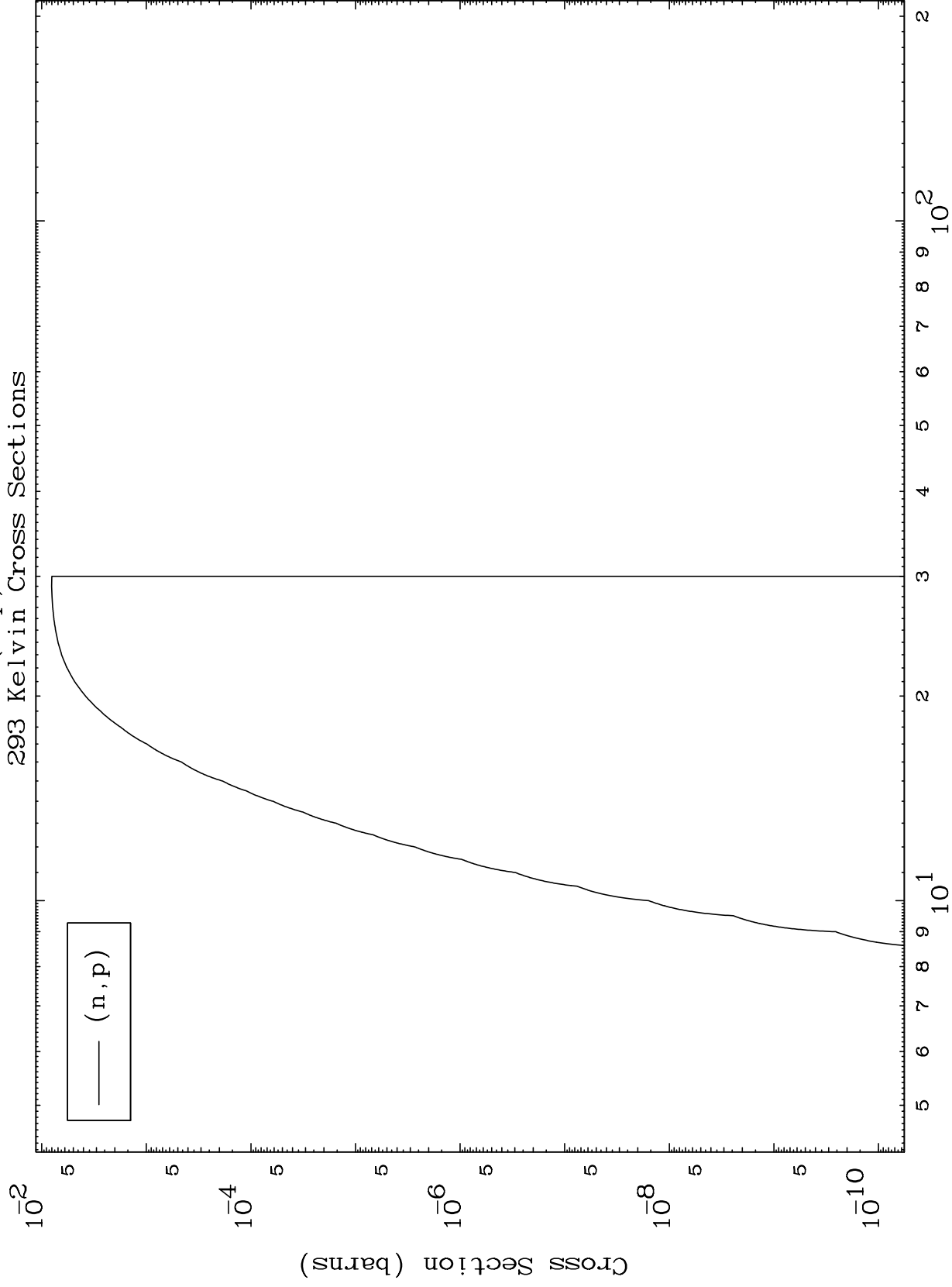
293 Kelvin Cross Sections



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(n,p) Levels
293 Kelvin Cross Sections

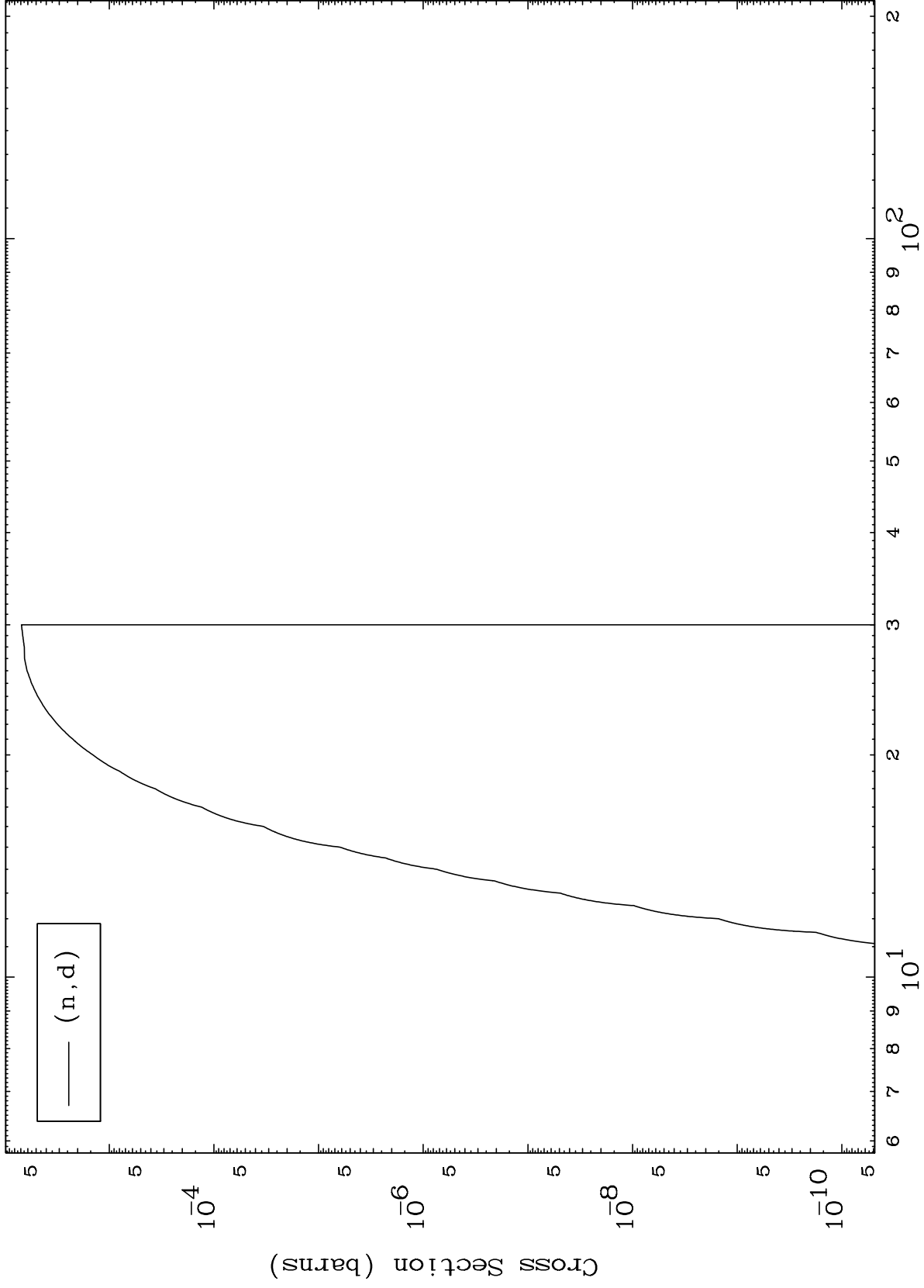
86-Rn-224



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(n,d) Levels
293 Kelvin Cross Sections

86-Rn-224



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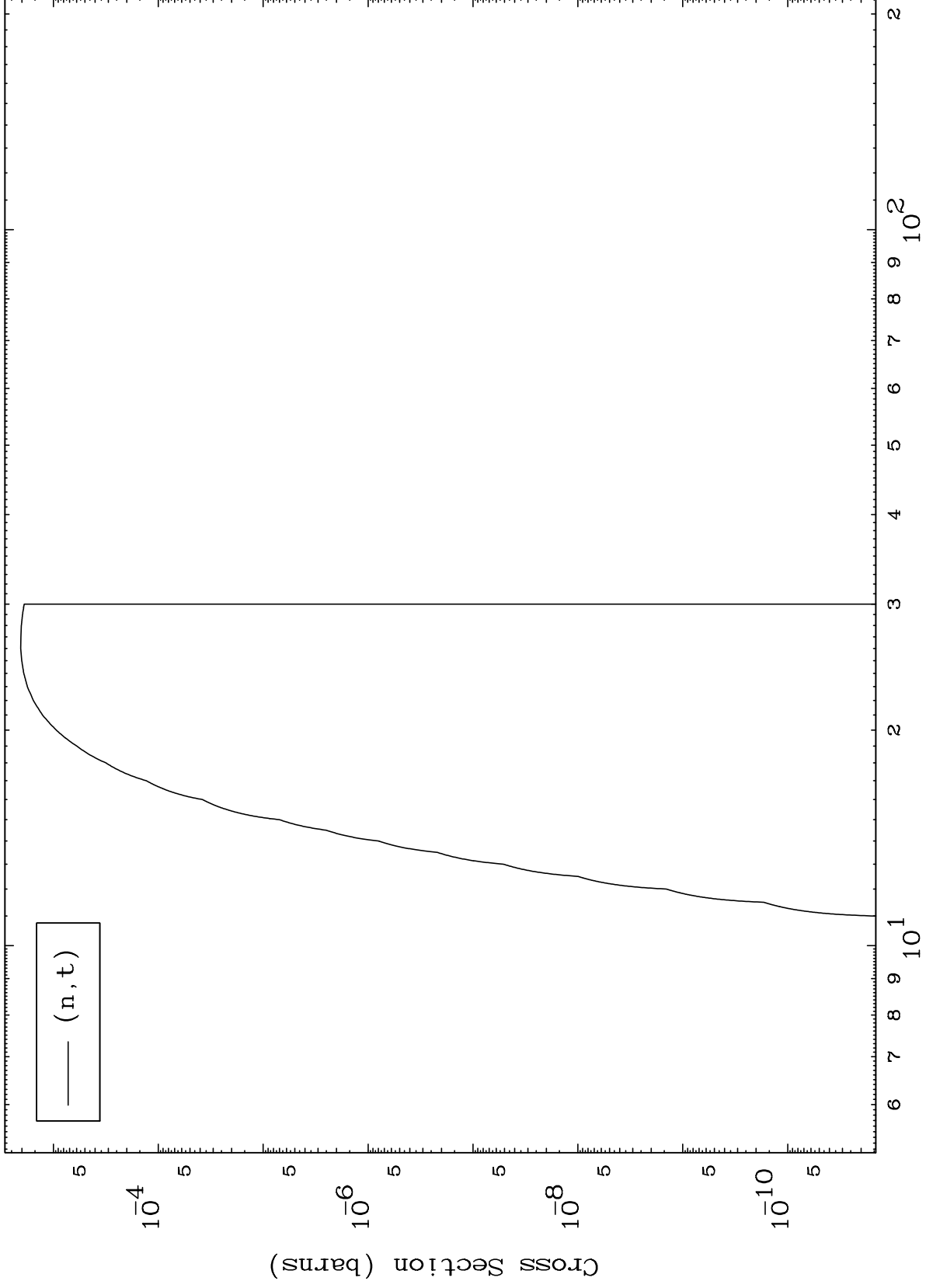
Incident Energy (MeV)

86-Rn-224

MAT 8664

(n, t) Levels
293 Kelvin Cross Sections

86-Rn-224



11

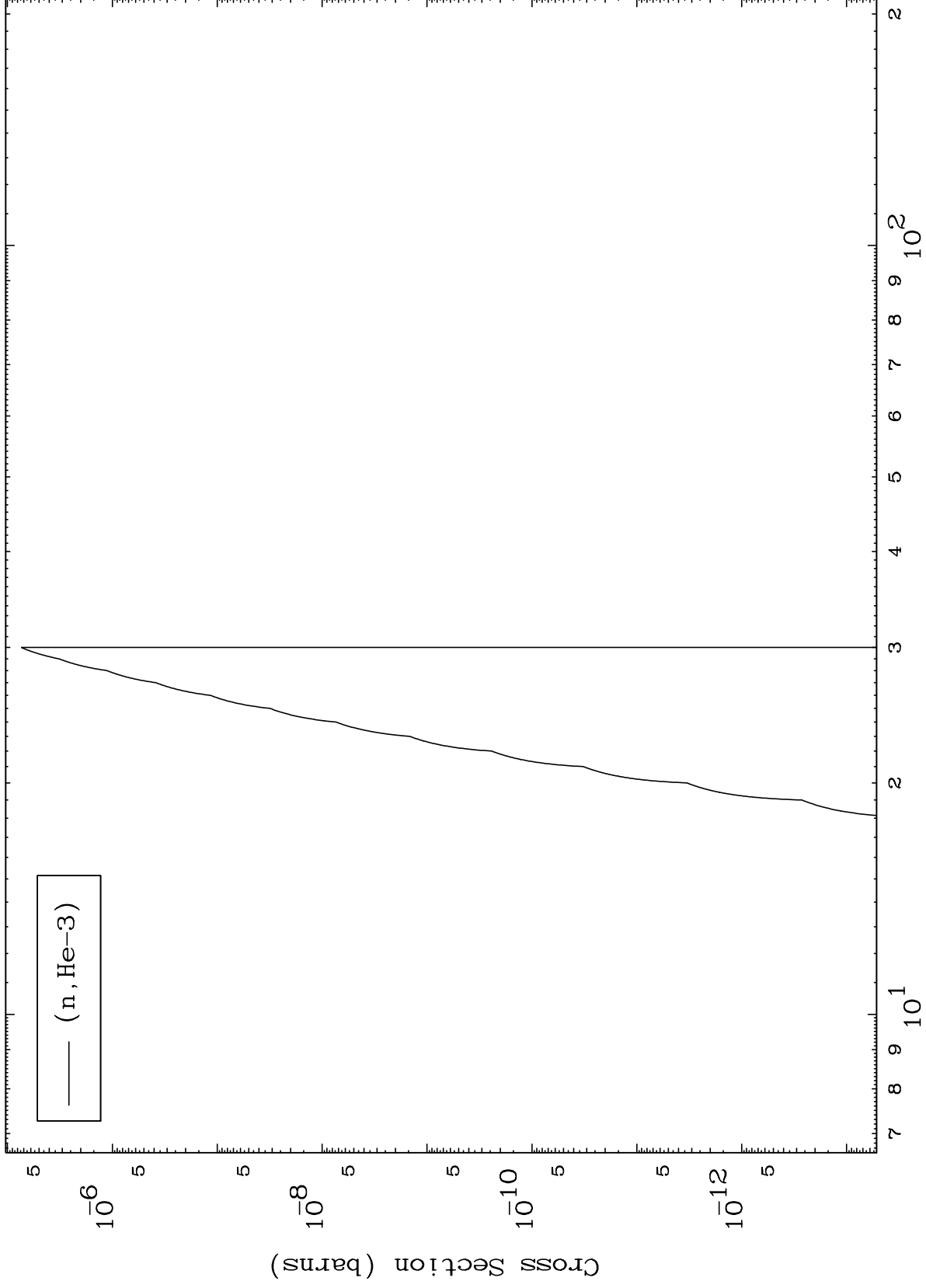
Incident Energy (MeV)

86-Rn-224

MAT 8664

(n,He3) Levels
293 Kelvin Cross Sections

86-Rn-224



12

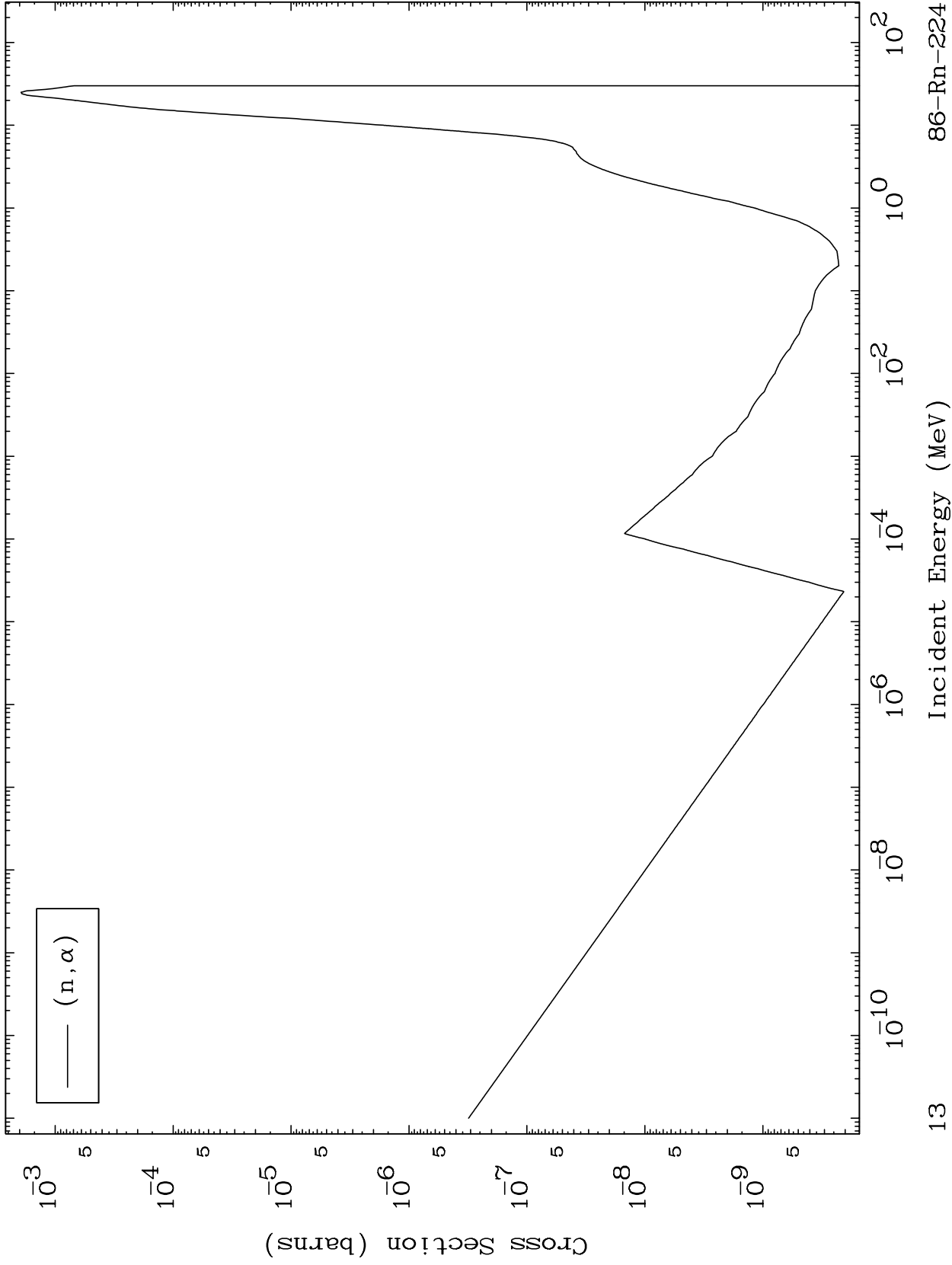
Incident Energy (MeV)

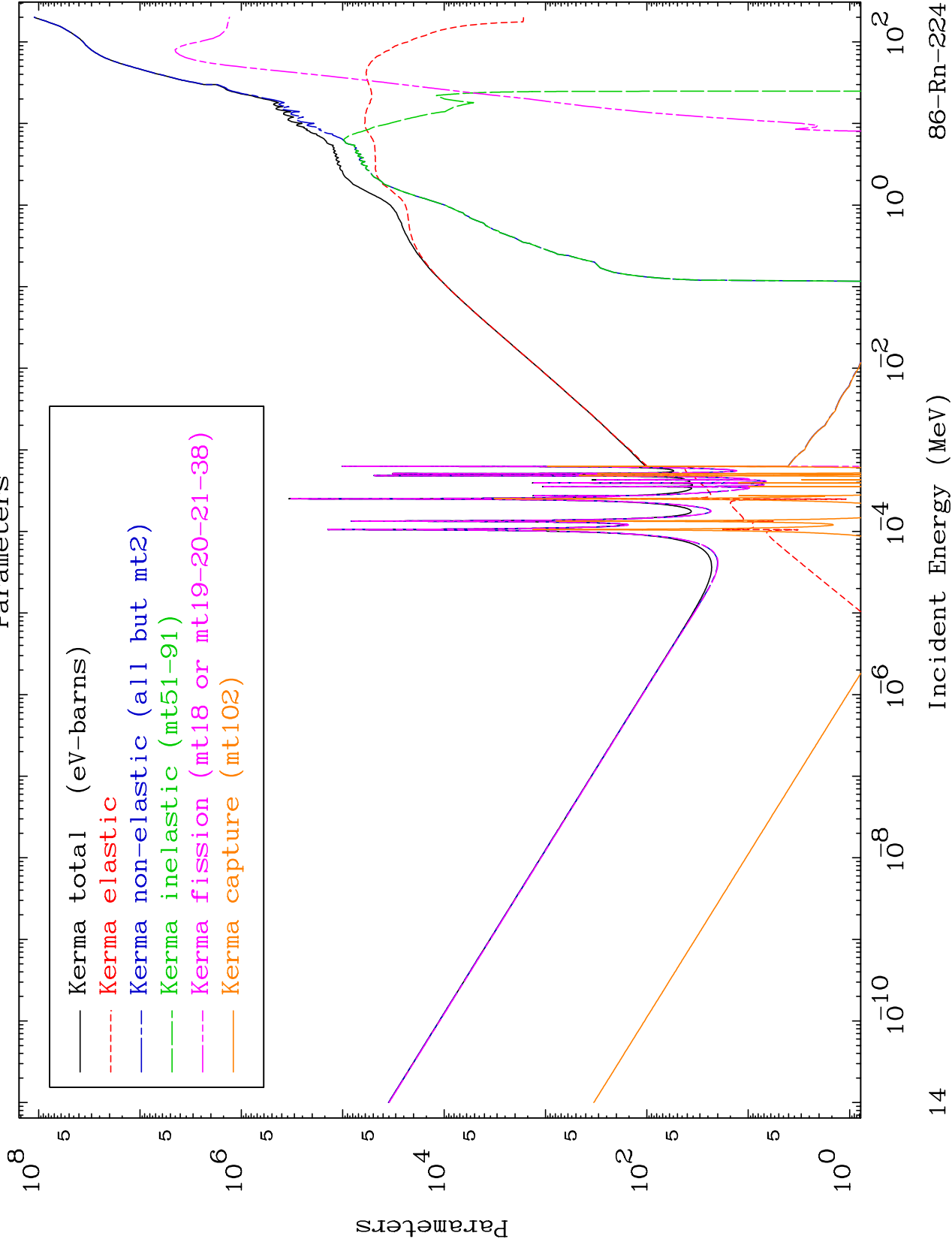
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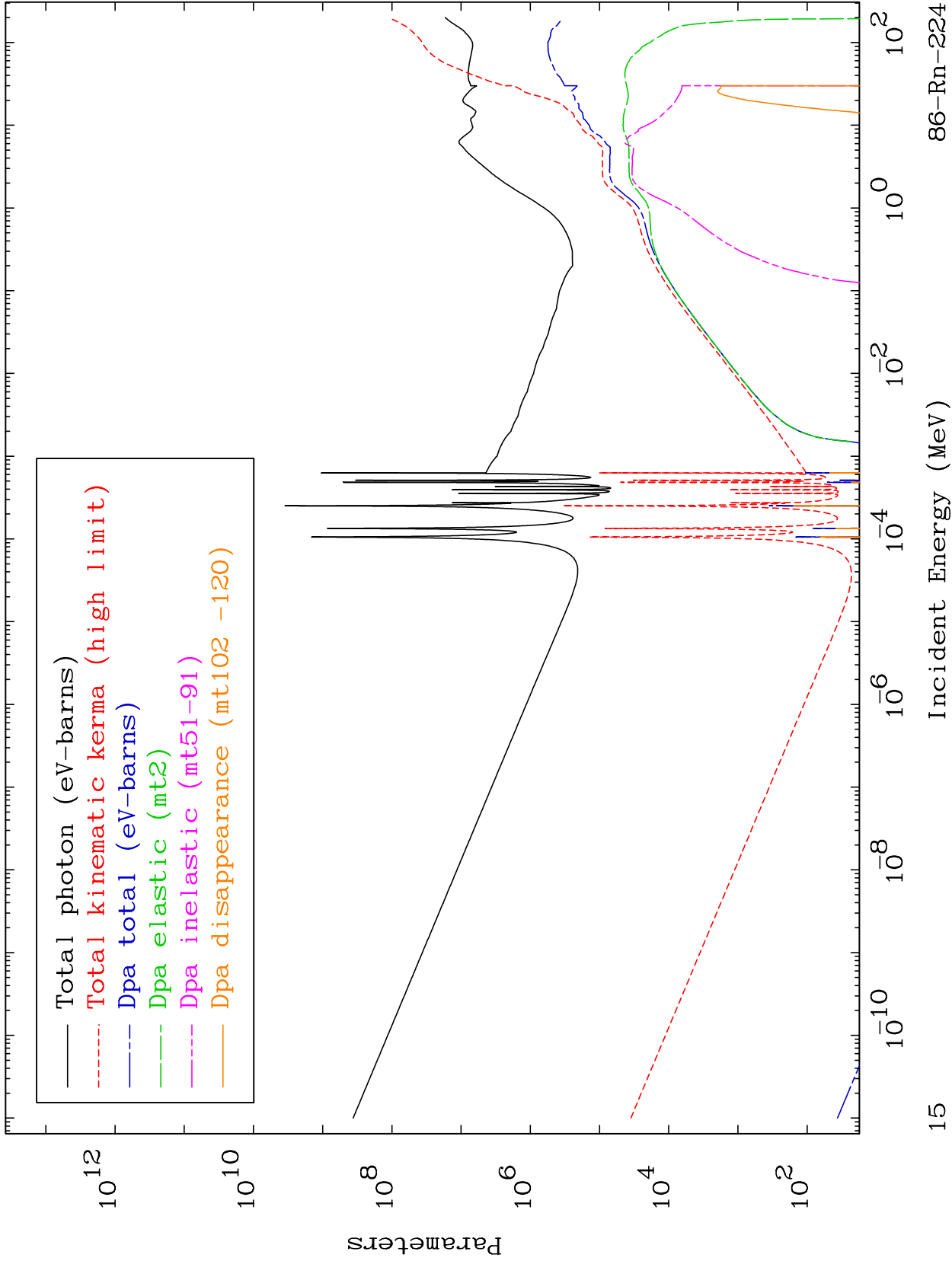
MAT 8664

(n, α) Levels
293 Kelvin Cross Sections

86-Rn-224



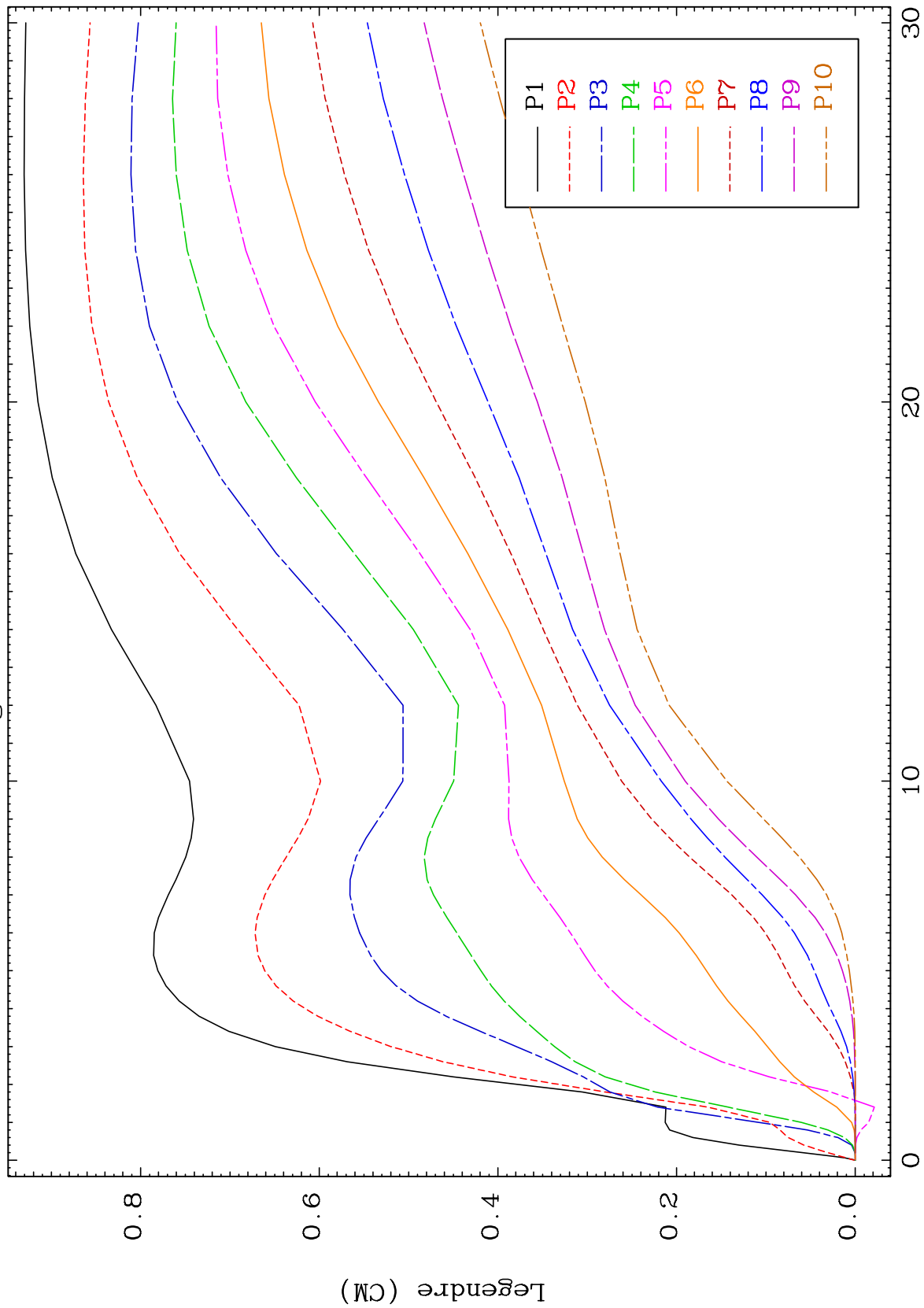




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Elastic Legendre Coefficients

86-Rn-224



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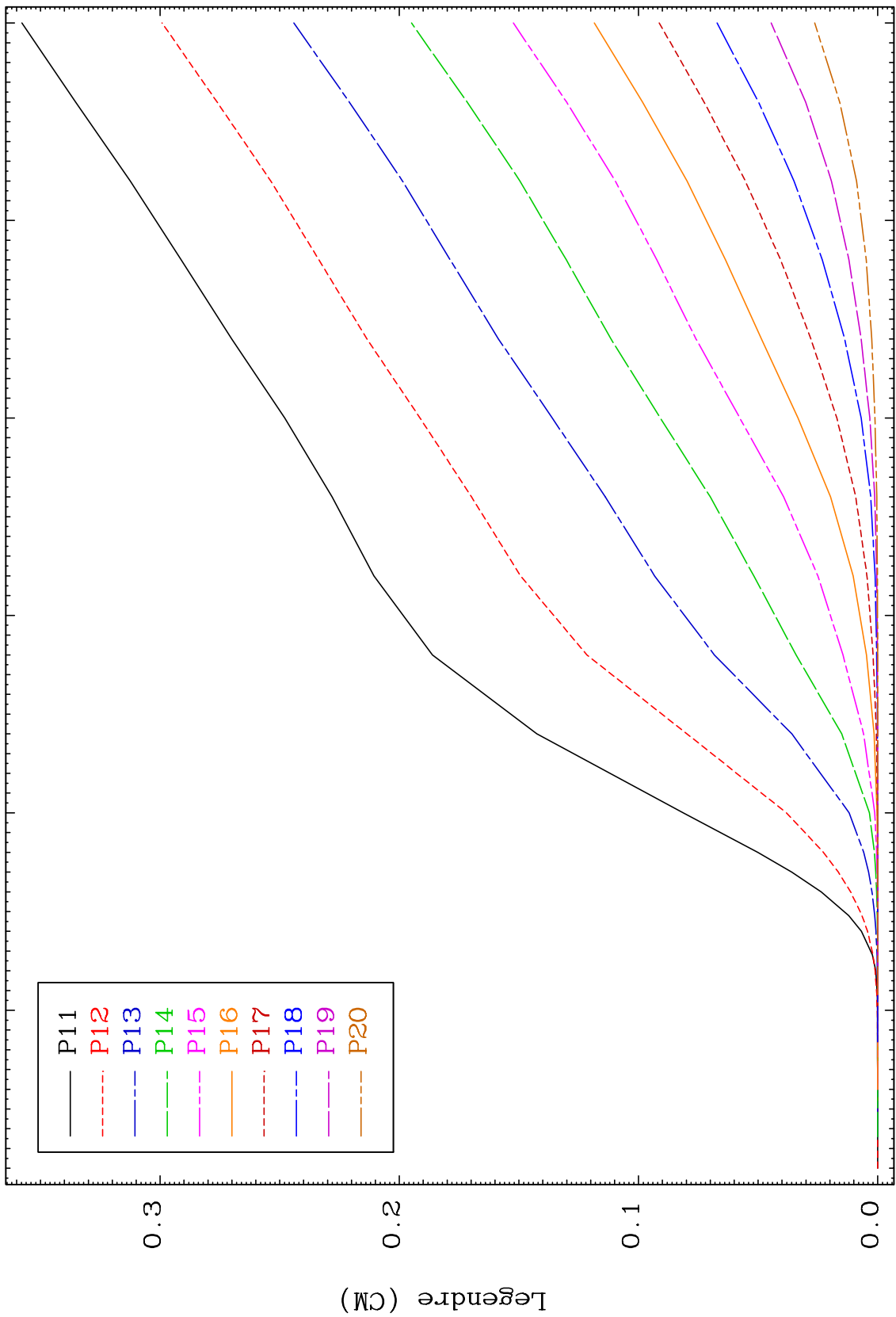
Incident Energy (MeV)

86-Rn-224

MAT 8664

Elastic Legendre Coefficients

86-Rn-224



17

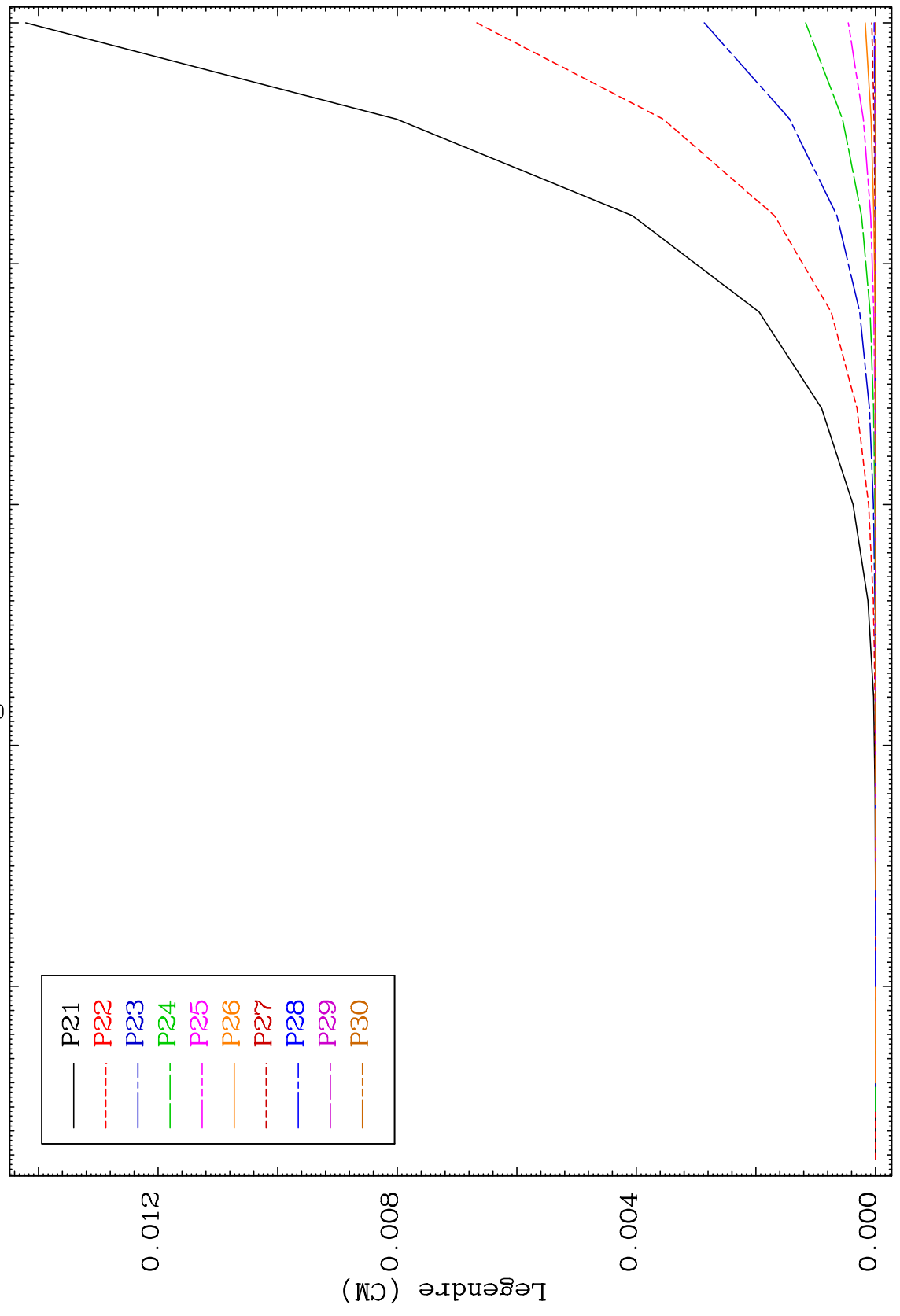
Incident Energy (MeV)

86-Rn-224

MAT 8664

Elastic Legendre Coefficients

86-Rn-224



18

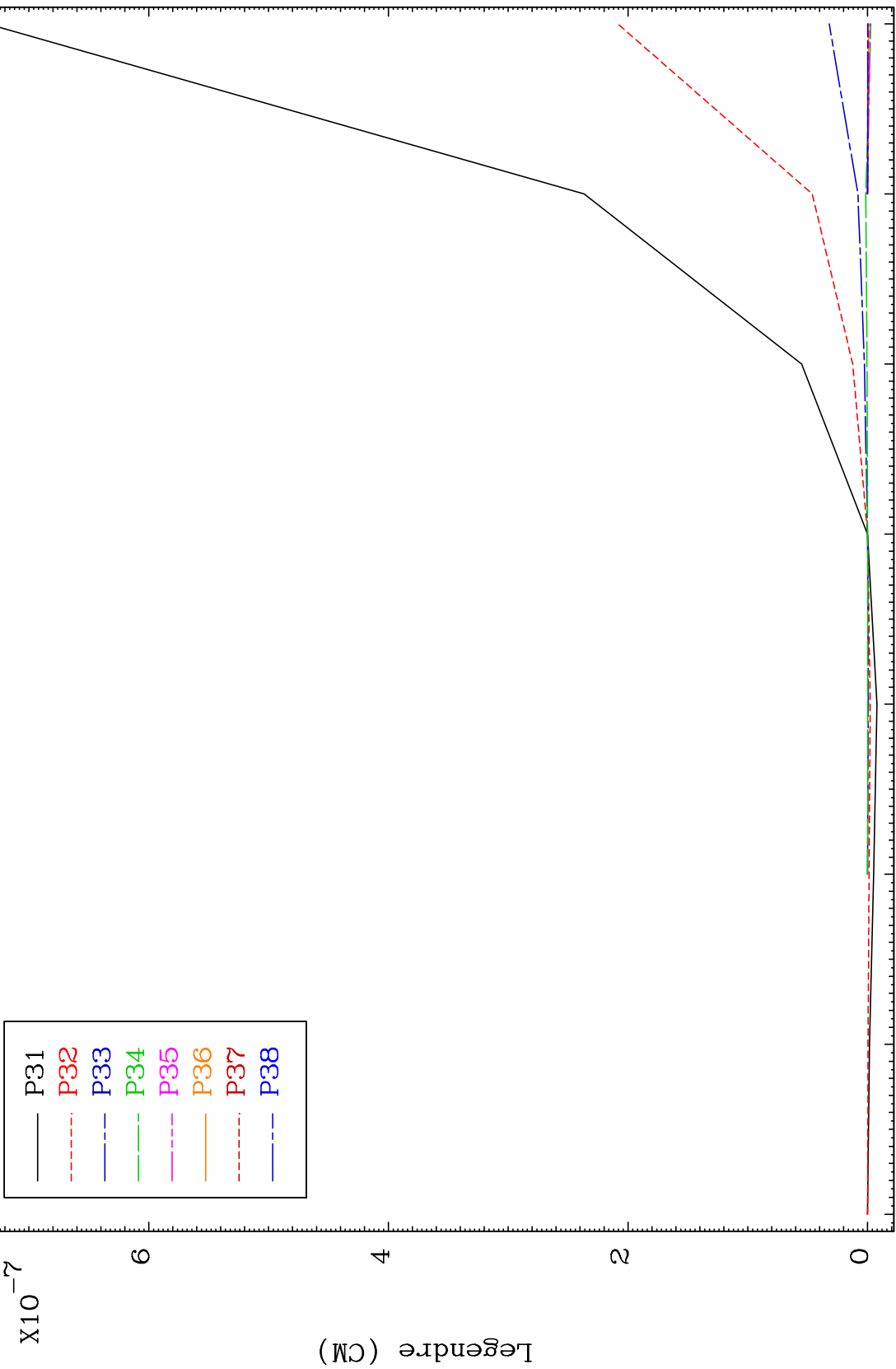
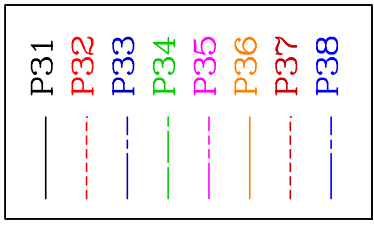
Incident Energy (MeV)

86-Rn-224

MAT 8664

Elastic Legendre Coefficients

⁸⁶Rn-²²⁴

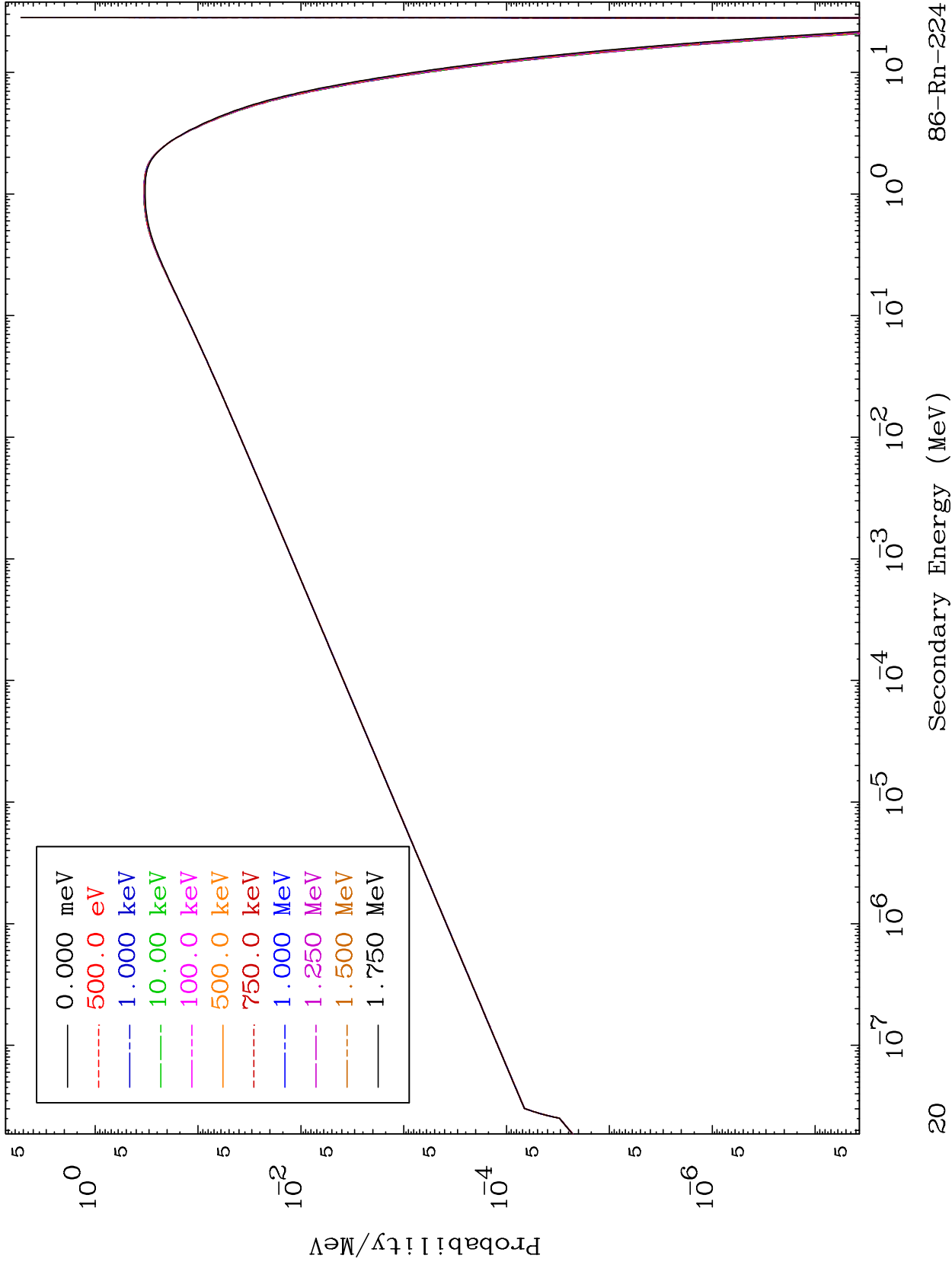


19

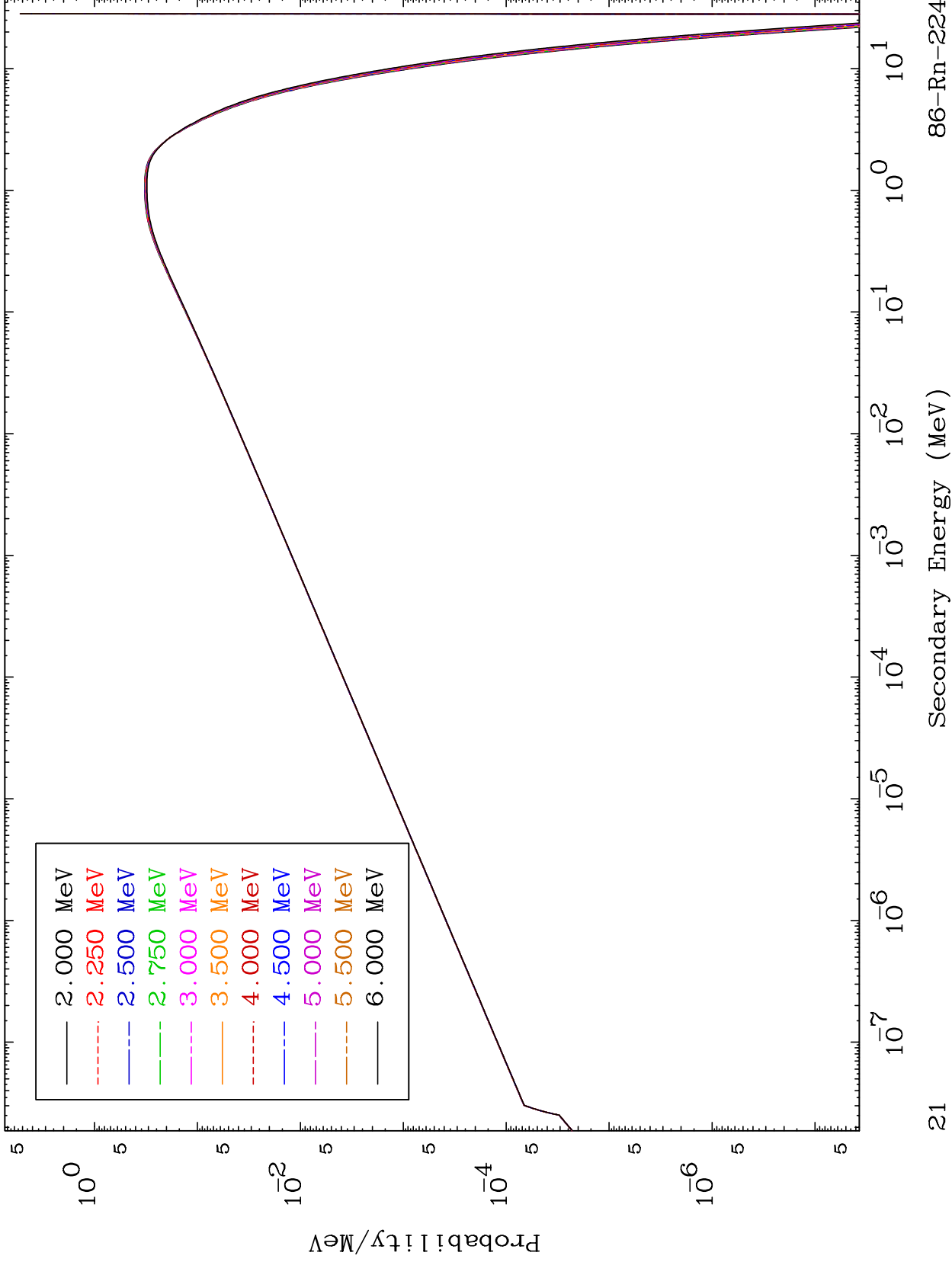
Incident Energy (MeV)

⁸⁶Rn-²²⁴

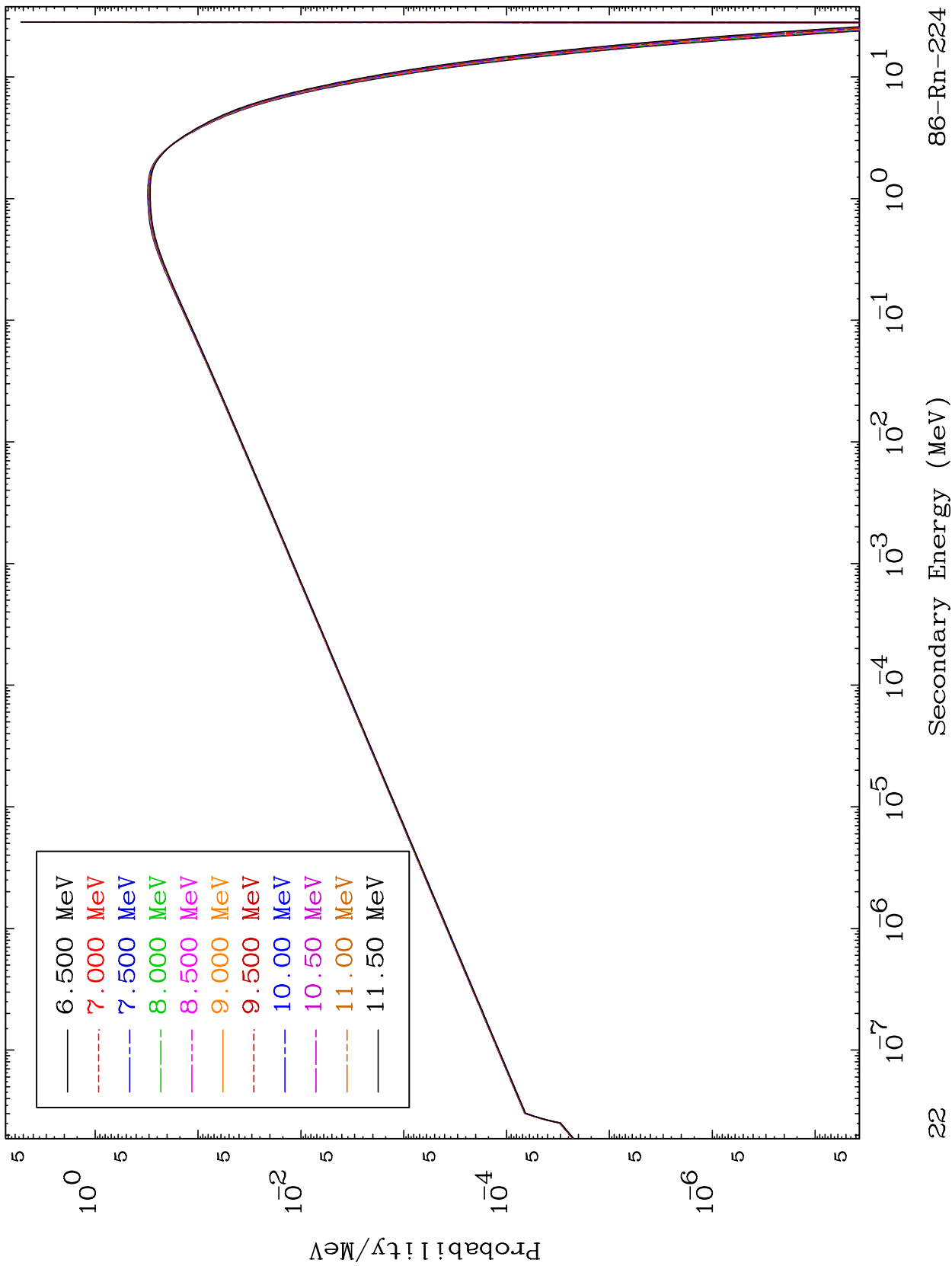
Fission Energy Distribution



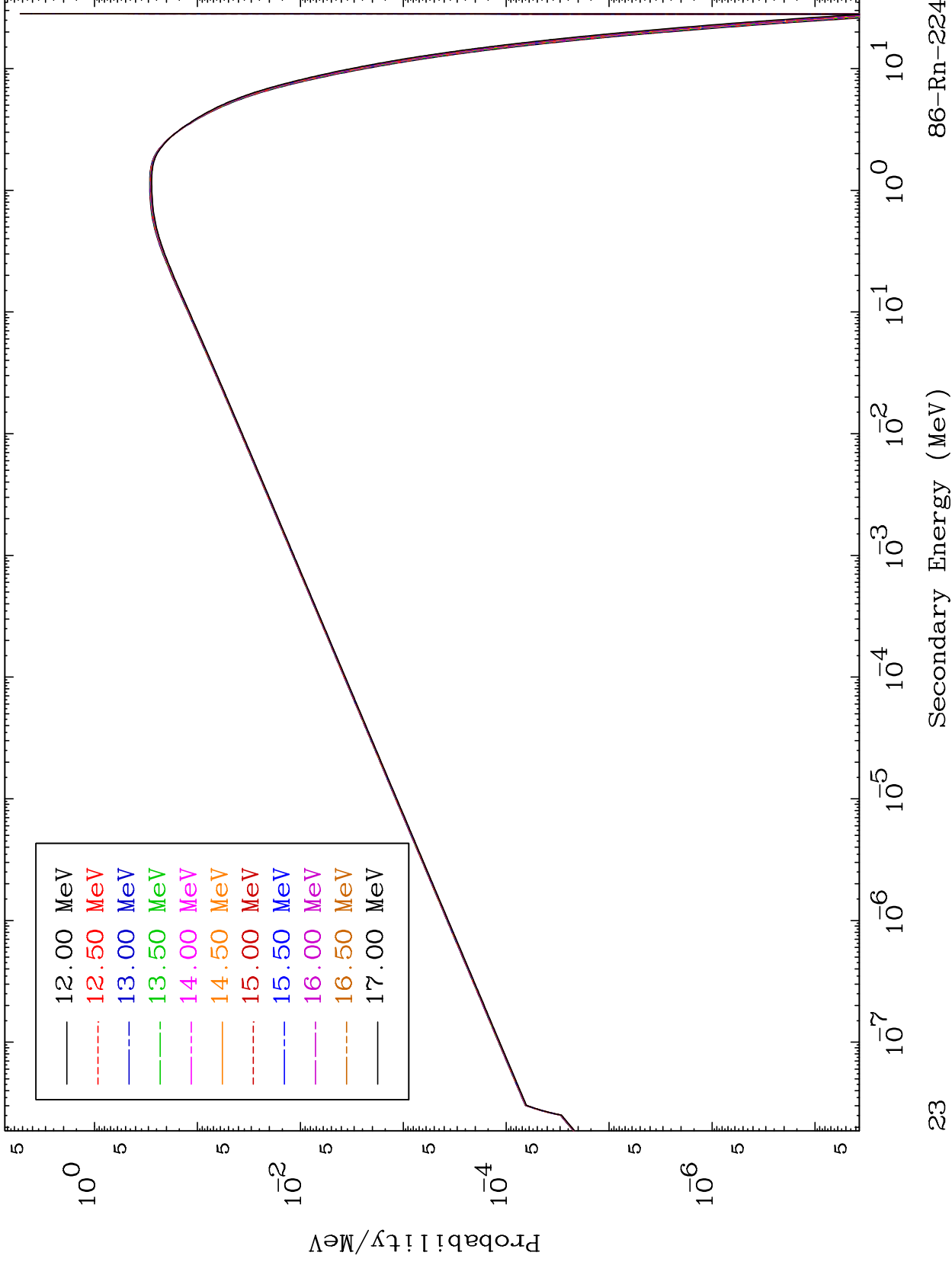
Fission
Energy Distribution



Fission Energy Distribution



Fission
Energy Distribution



Fission
Energy Distribution

