

Program EVALPLOT
(Version 2015-2)

by

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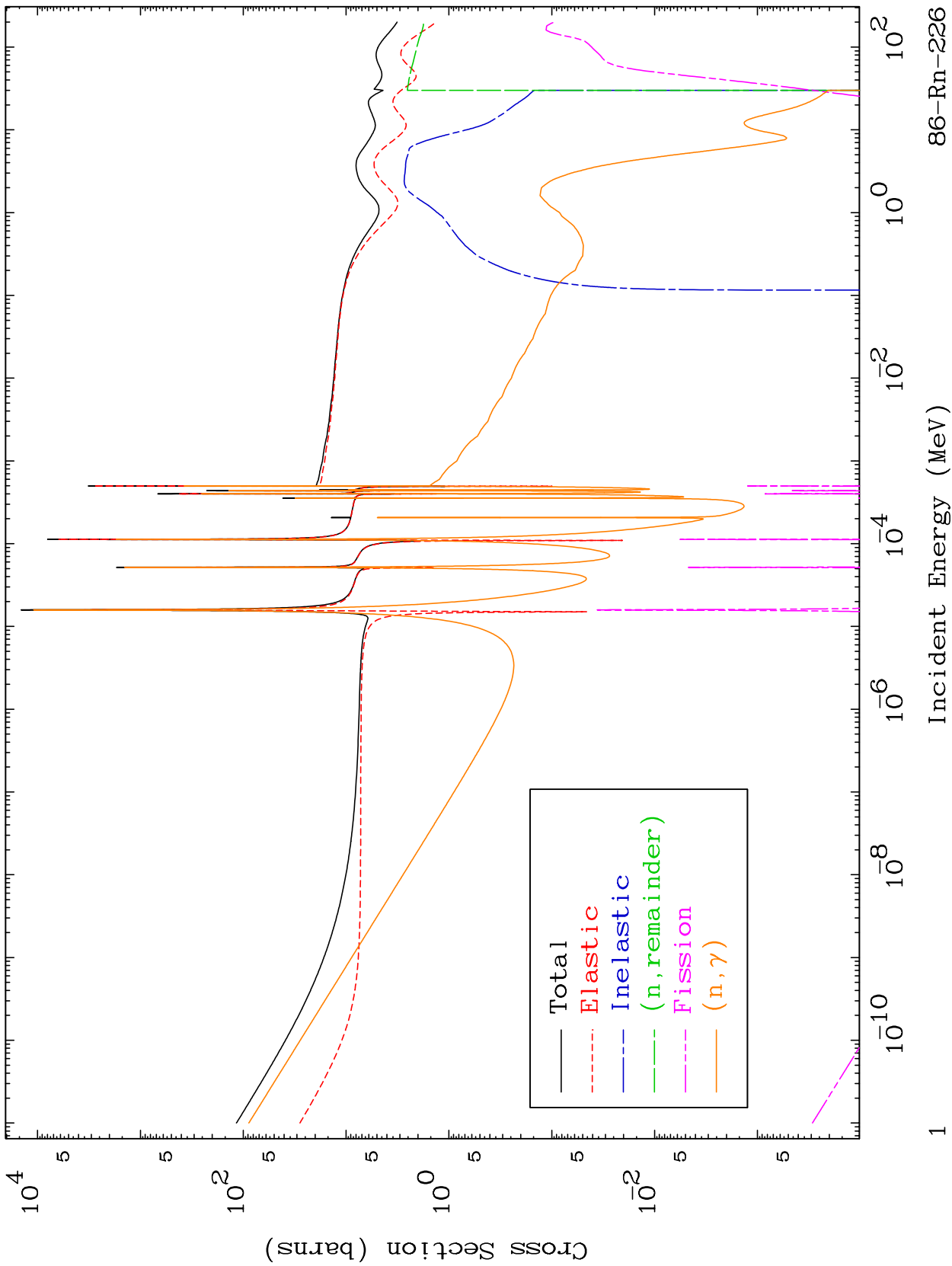
Press Mouse Button to Start

MAT 8670

Major

293 Kelvin Cross Sections

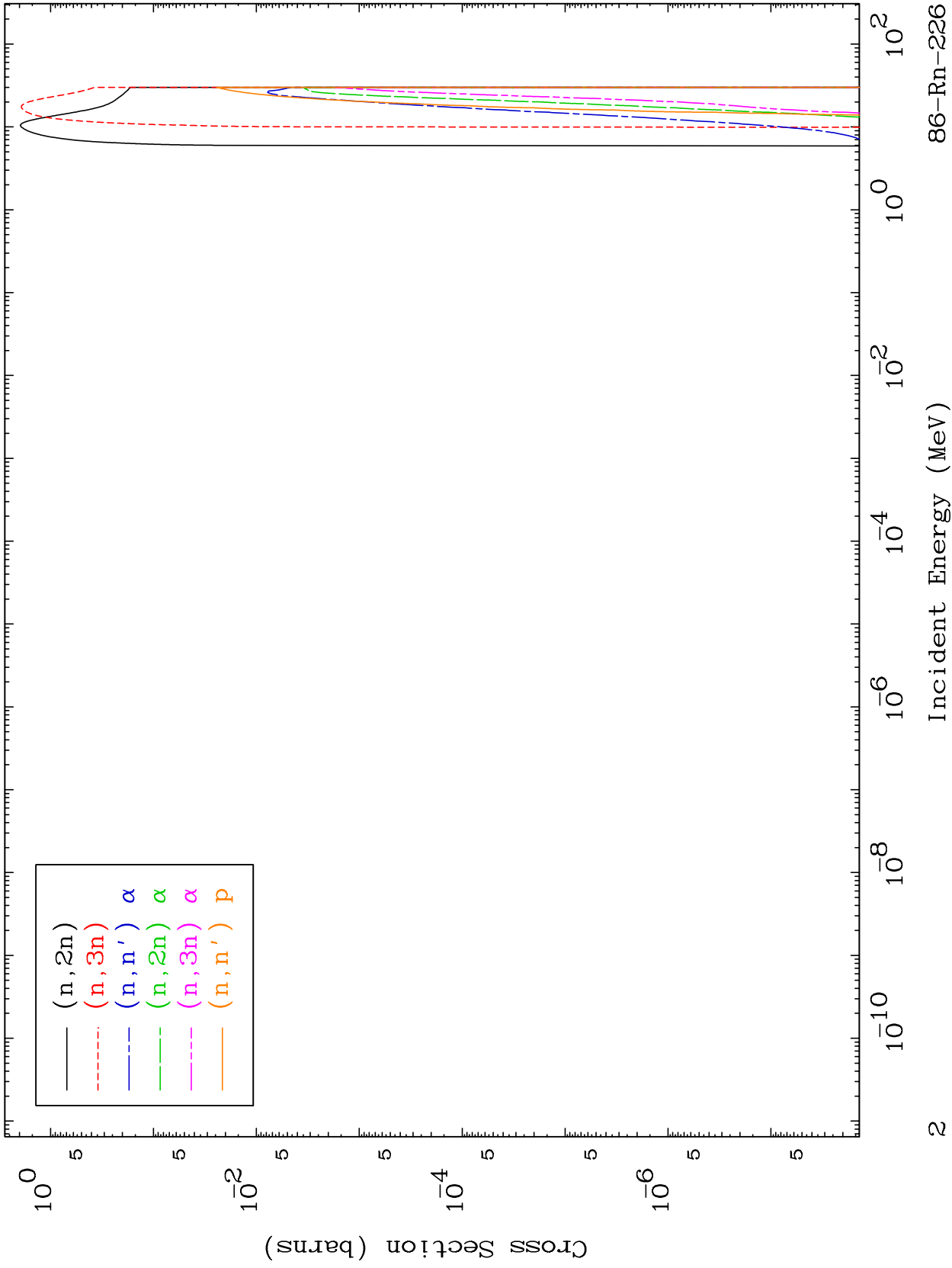
86-Rn-226

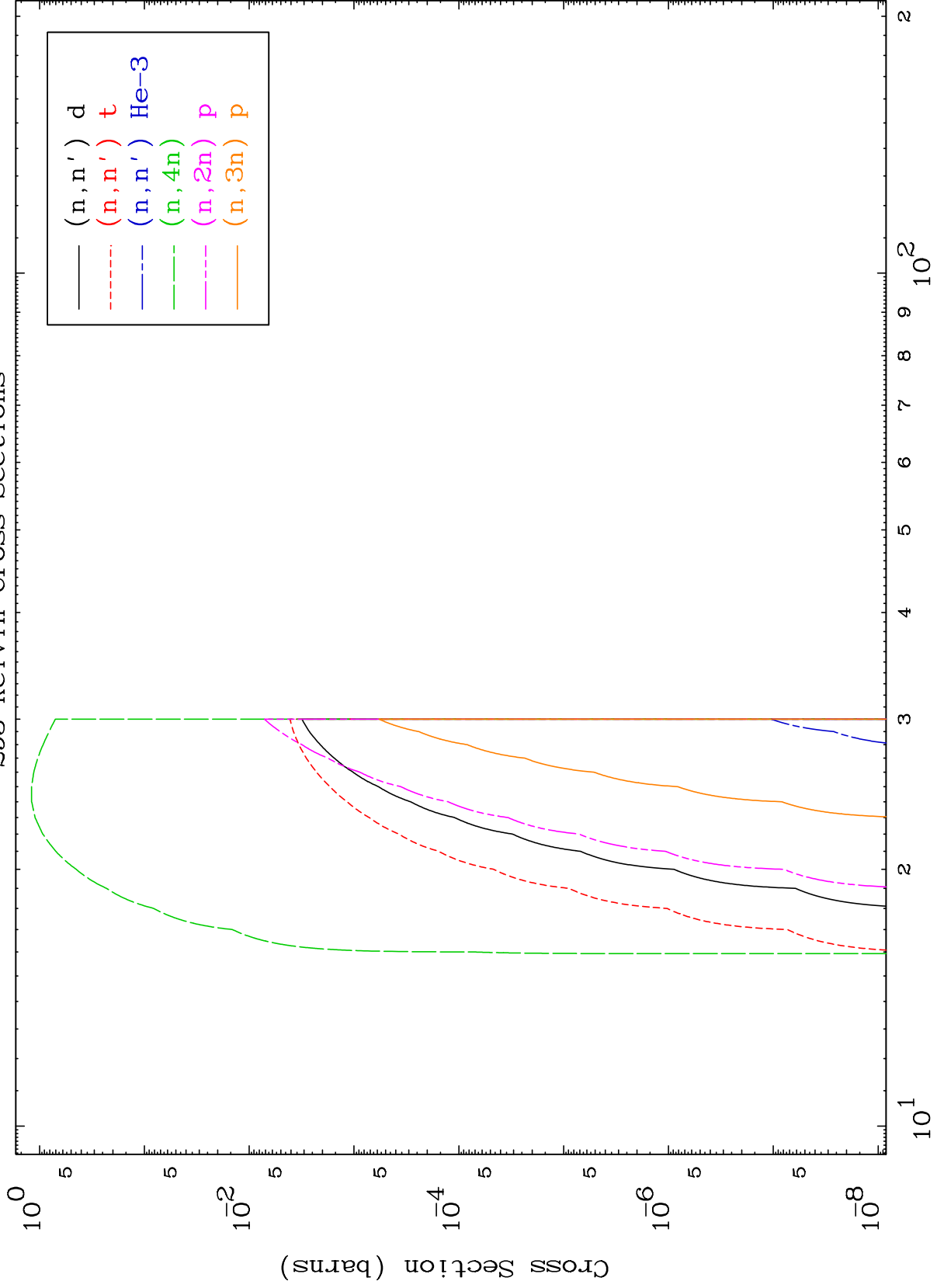


MAT 8670

Neutron Production
293 Kelvin Cross Sections

86-Rn-226

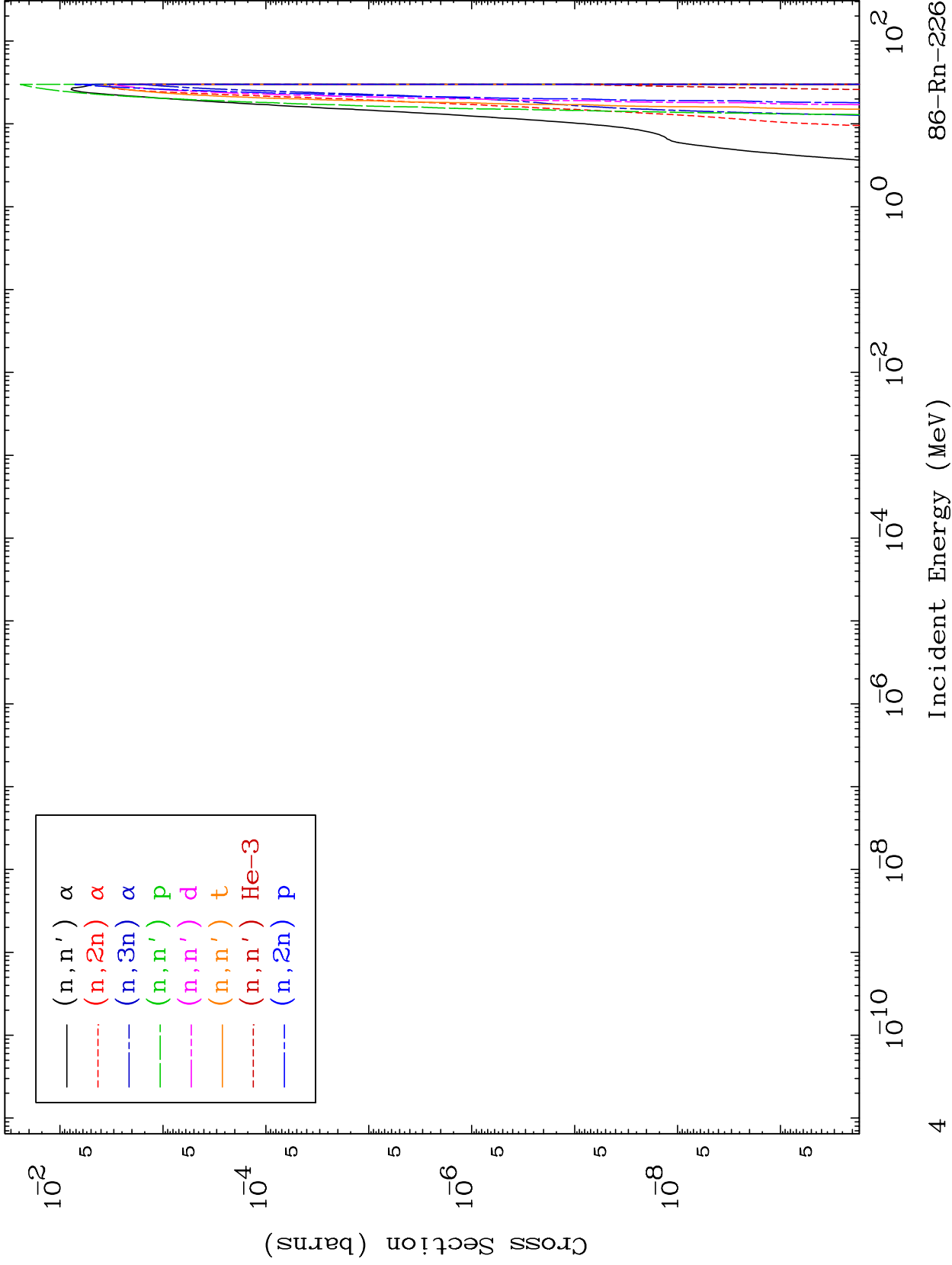




MAT 8670

Charged Particle
293 Kelvin Cross Sections

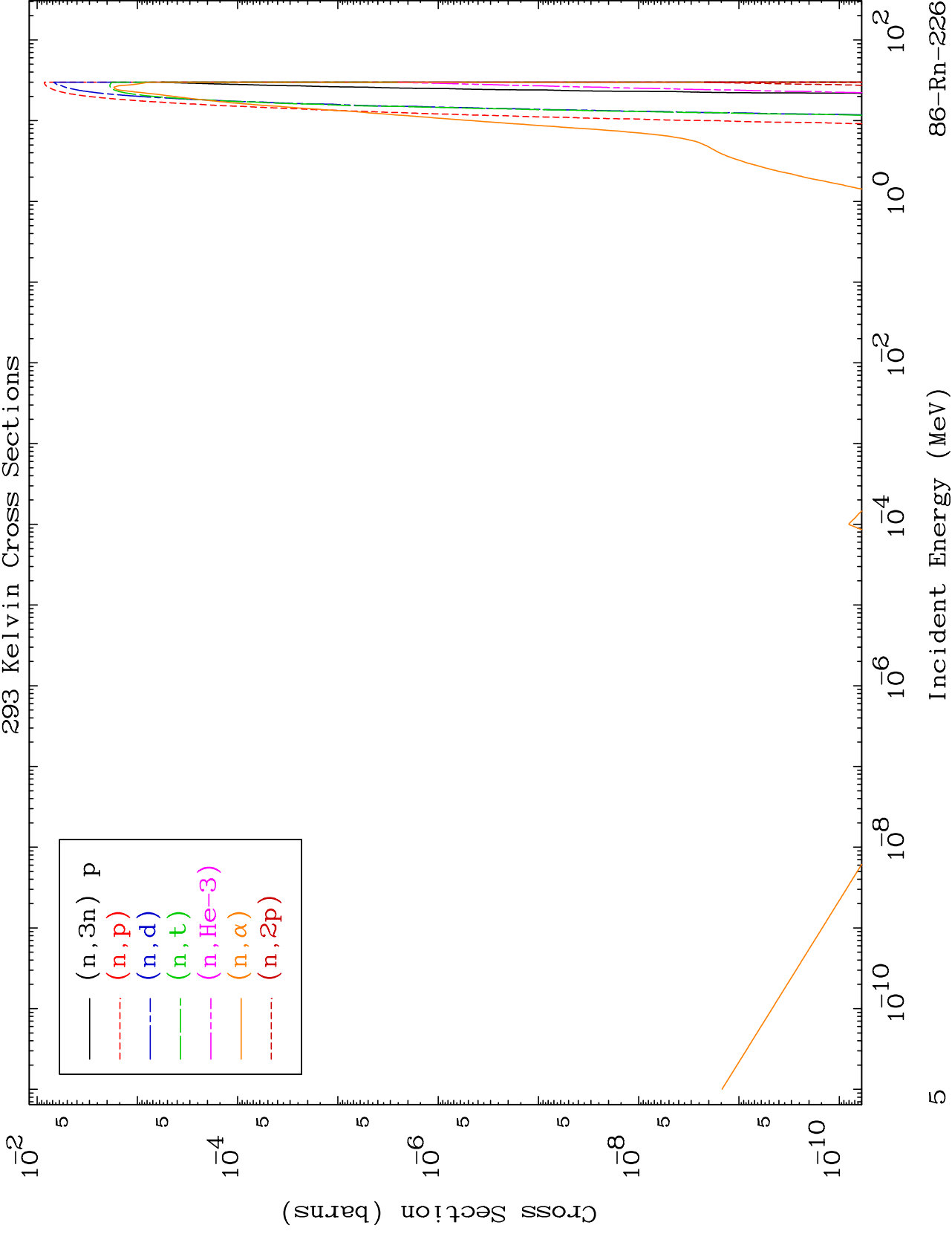
86-Rn-226



MAT 8670

Charged Particle
293 Kelvin Cross Sections

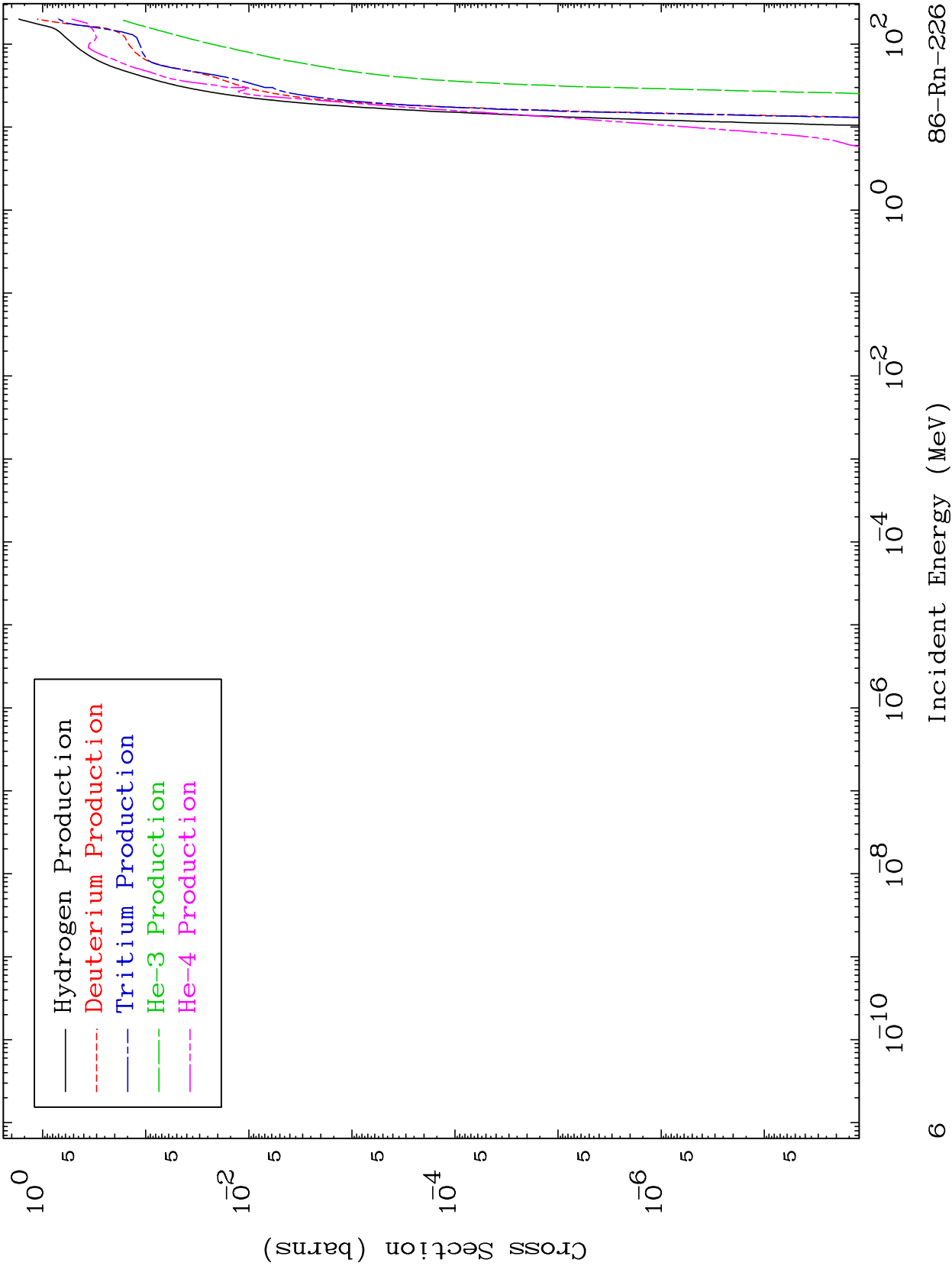
86-Rn-226



MAT 8670

Particle Production
293 Kelvin Cross Sections

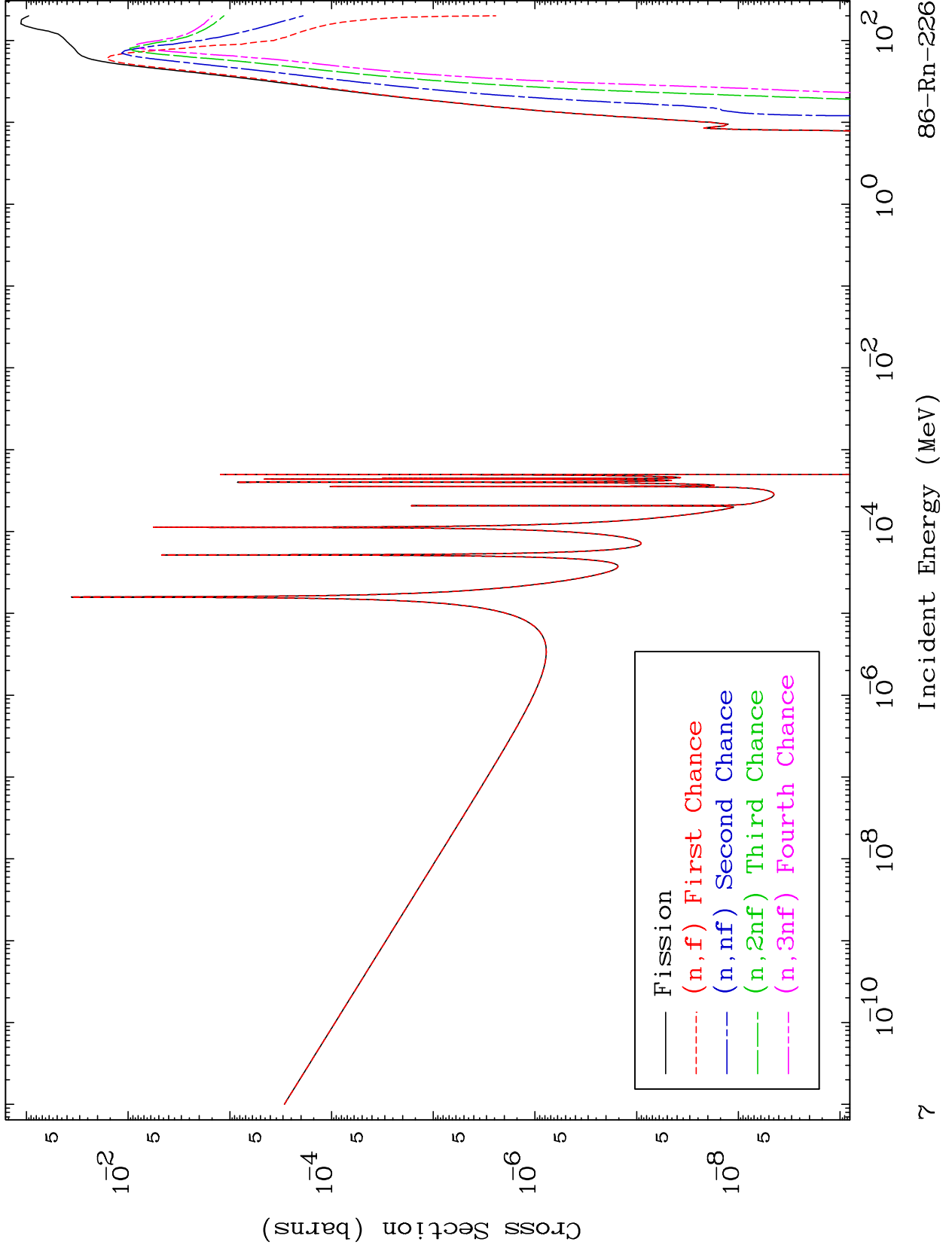
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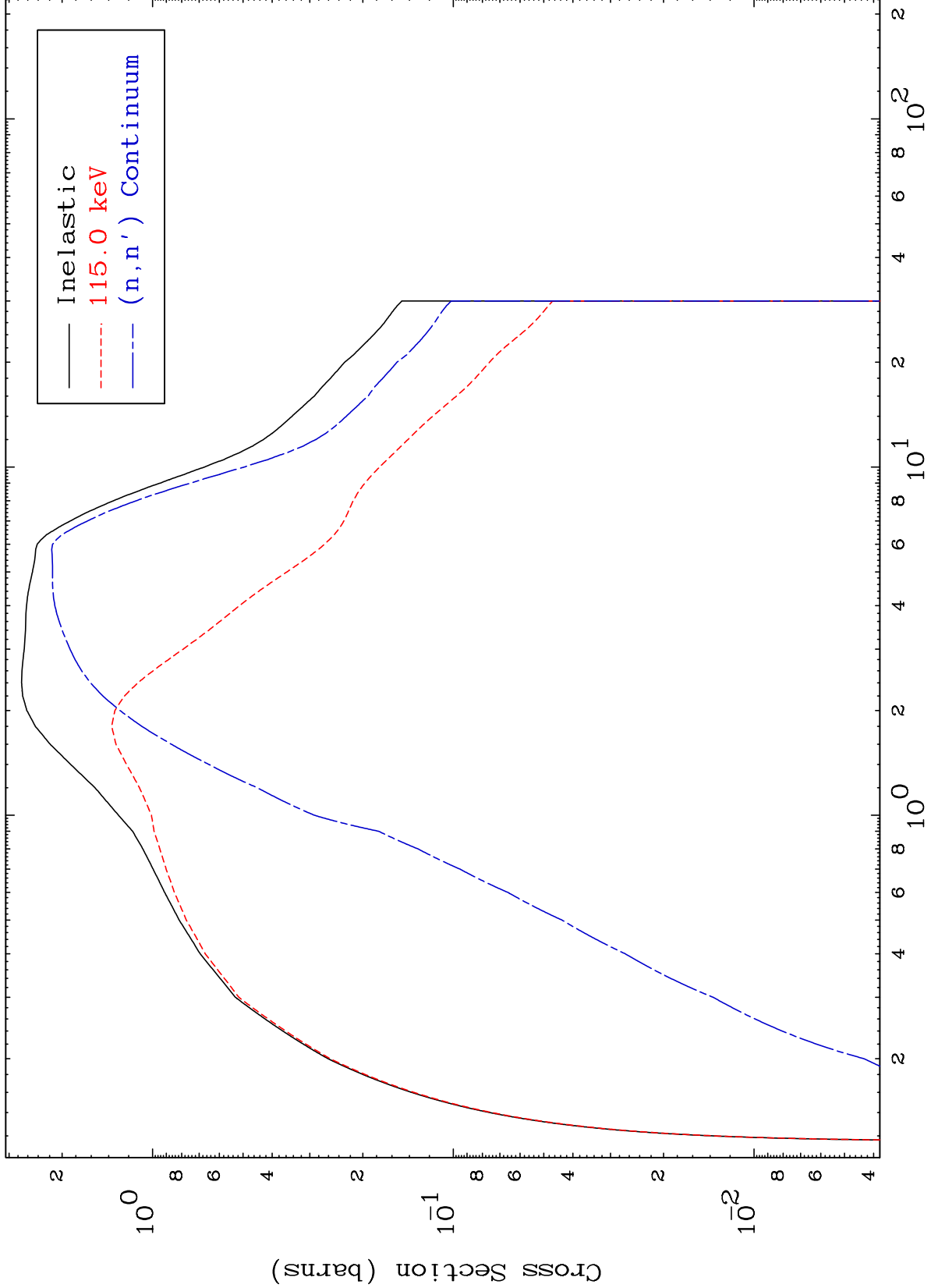
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Fission
293 Kelvin Cross Sections

86-Rn-226



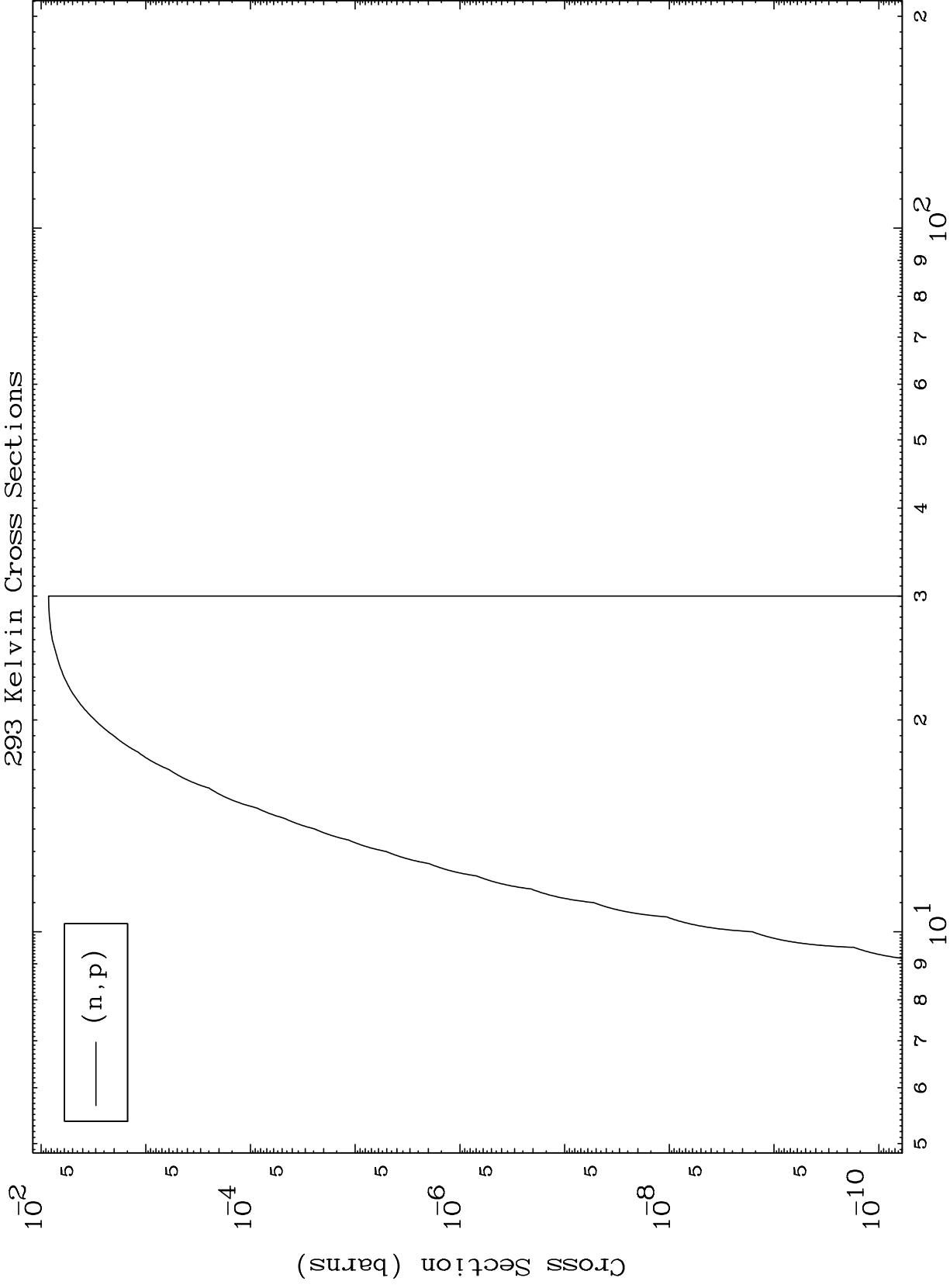
293 Kelvin Cross Sections



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(n,p) Levels
293 Kelvin Cross Sections

86-Rn-226



9

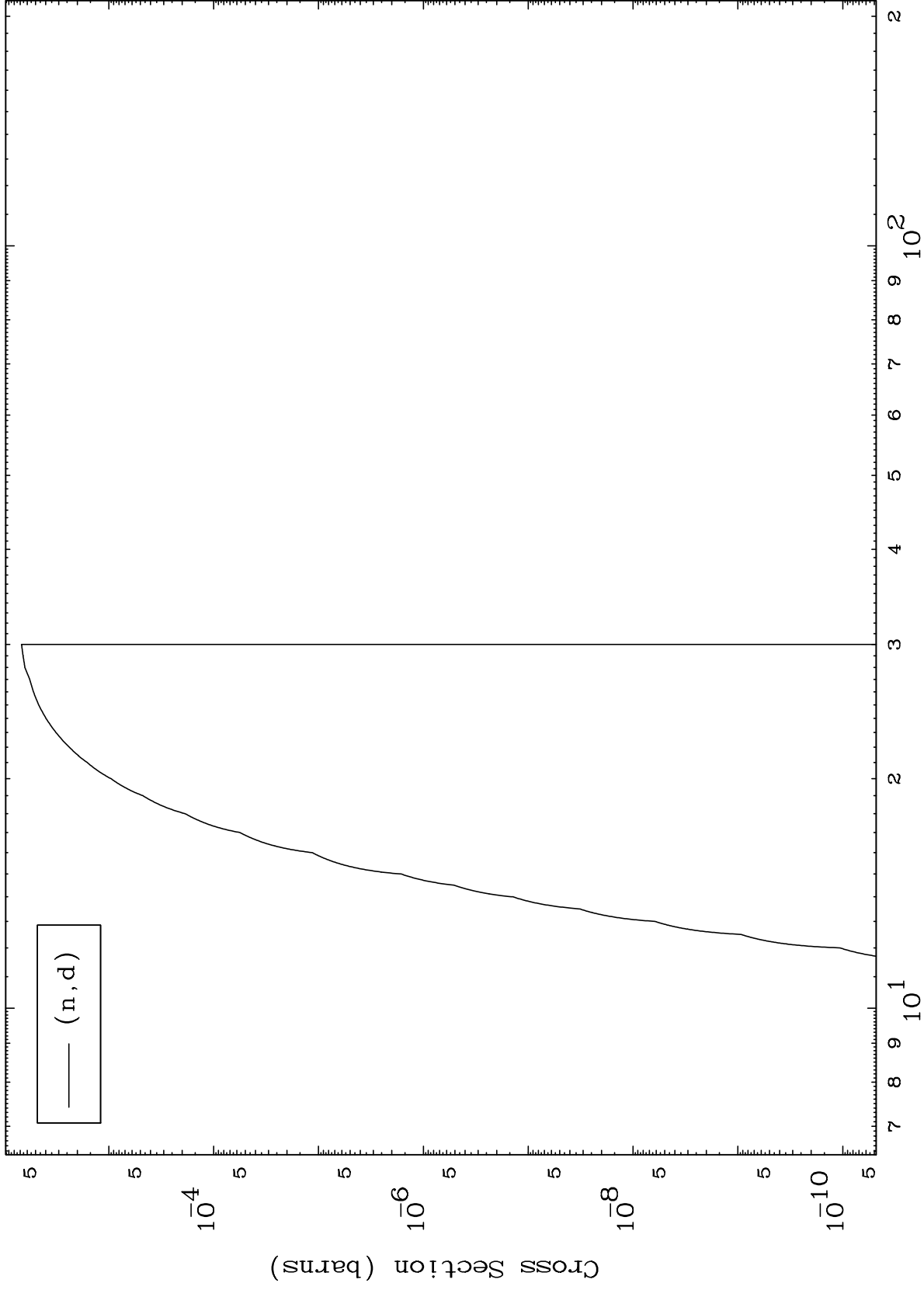
Incident Energy (MeV)

86-Rn-226

MAT 8670

(n,d) Levels
293 Kelvin Cross Sections

86-Rn-226



10

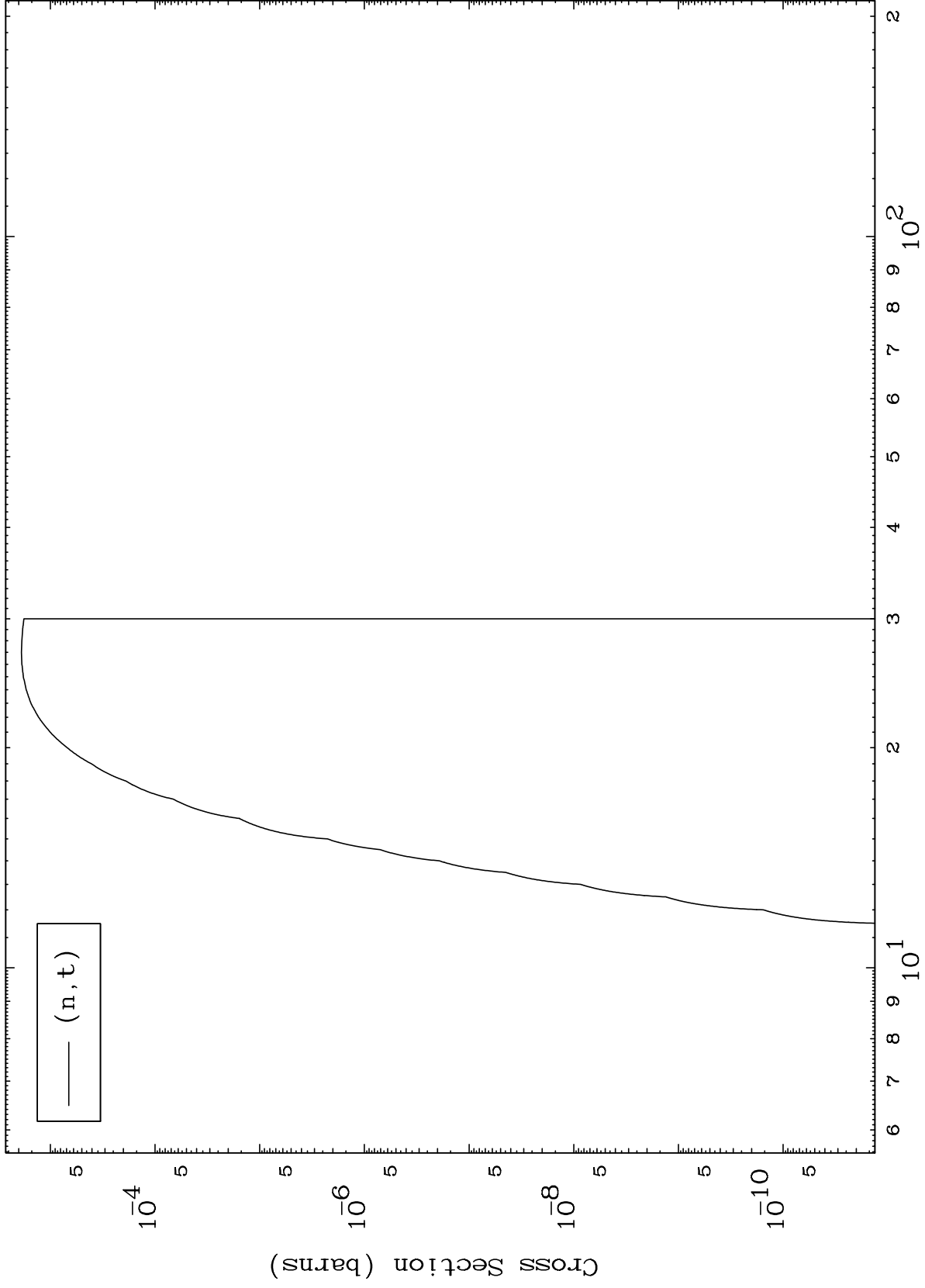
Incident Energy (MeV)

86-Rn-226

MAT 8670

(n, t) Levels
293 Kelvin Cross Sections

86-Rn-226



11

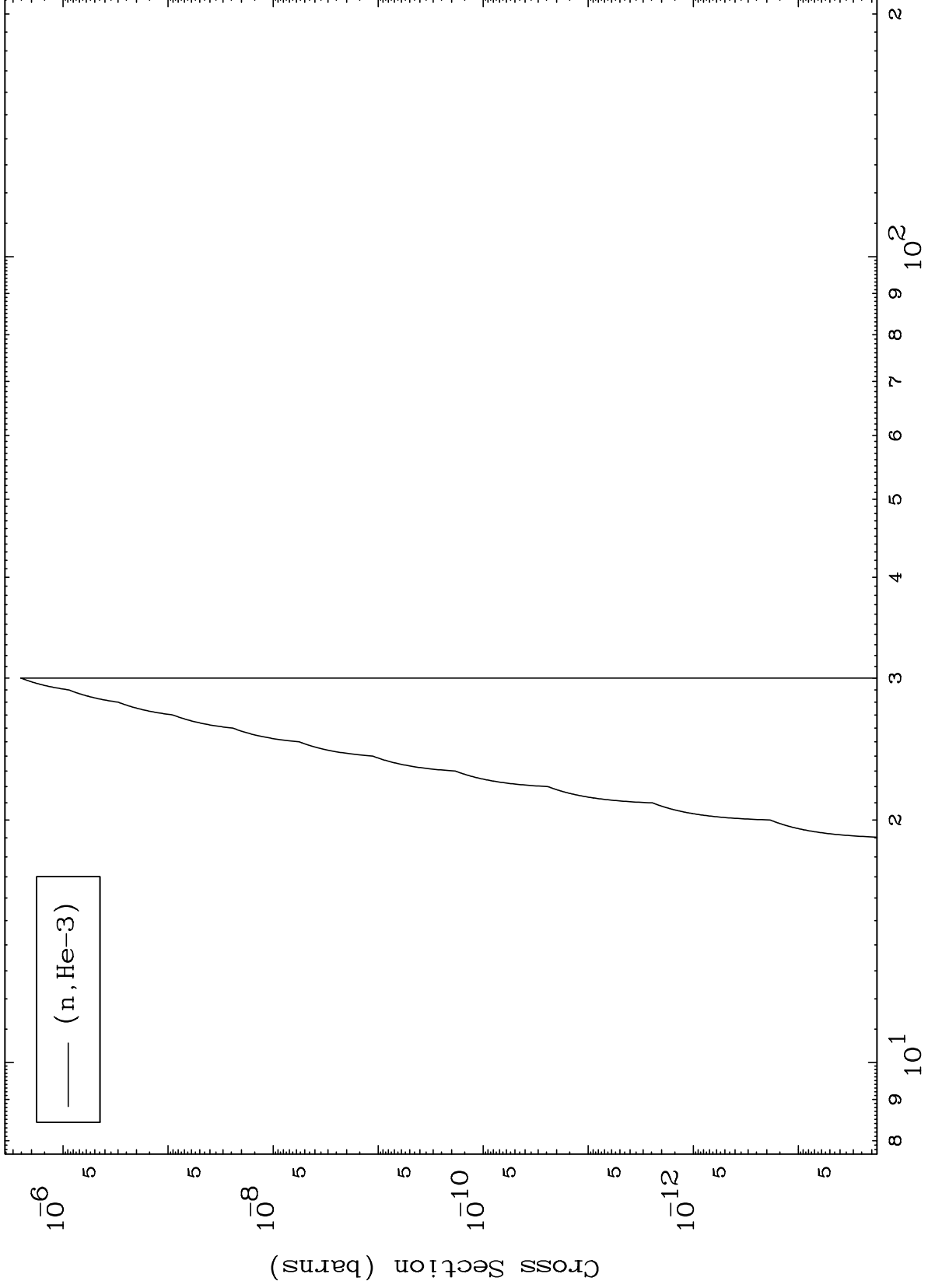
Incident Energy (MeV)

86-Rn-226

MAT 8670

(n,He3) Levels
293 Kelvin Cross Sections

86-Rn-226



12

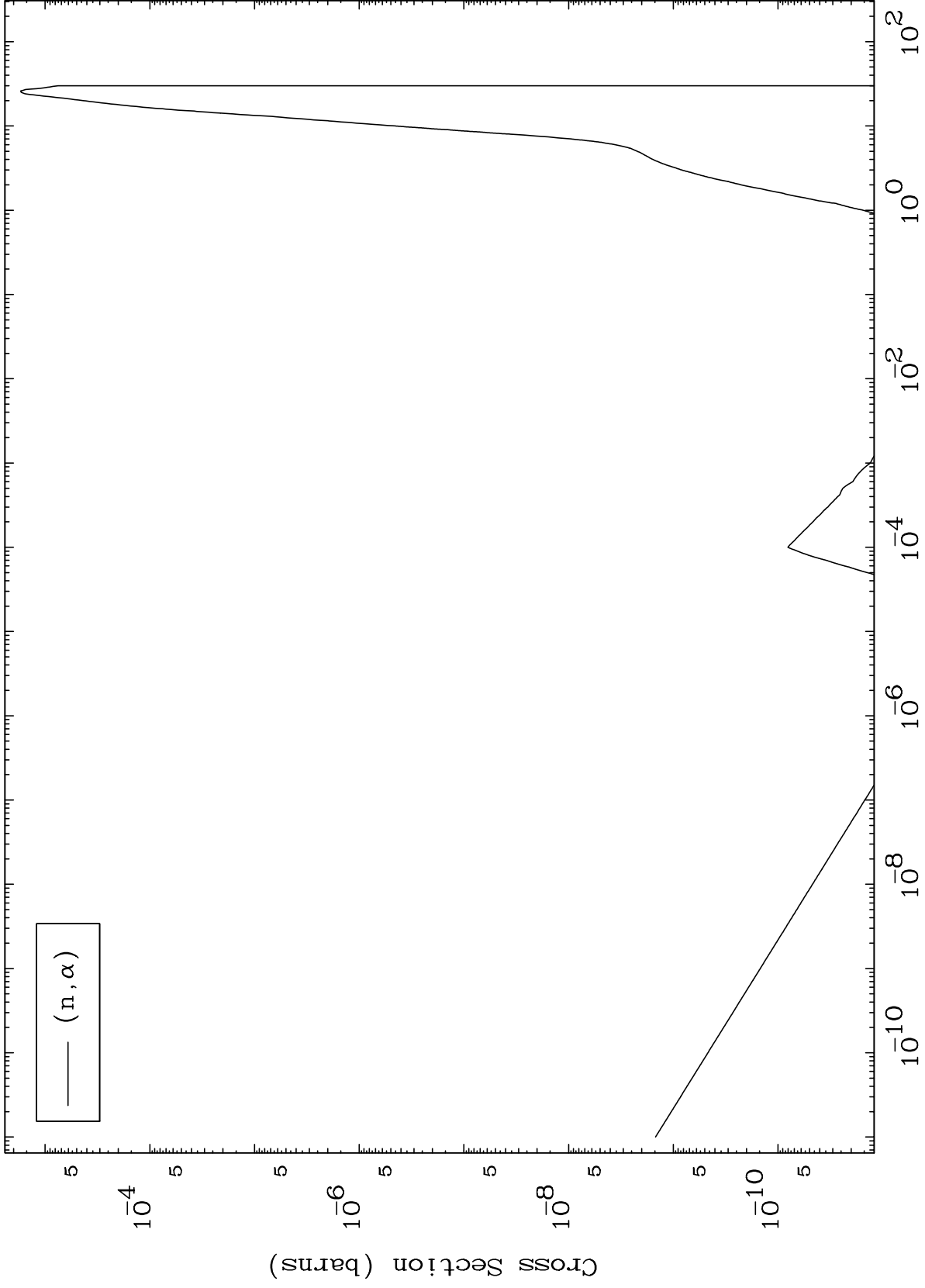
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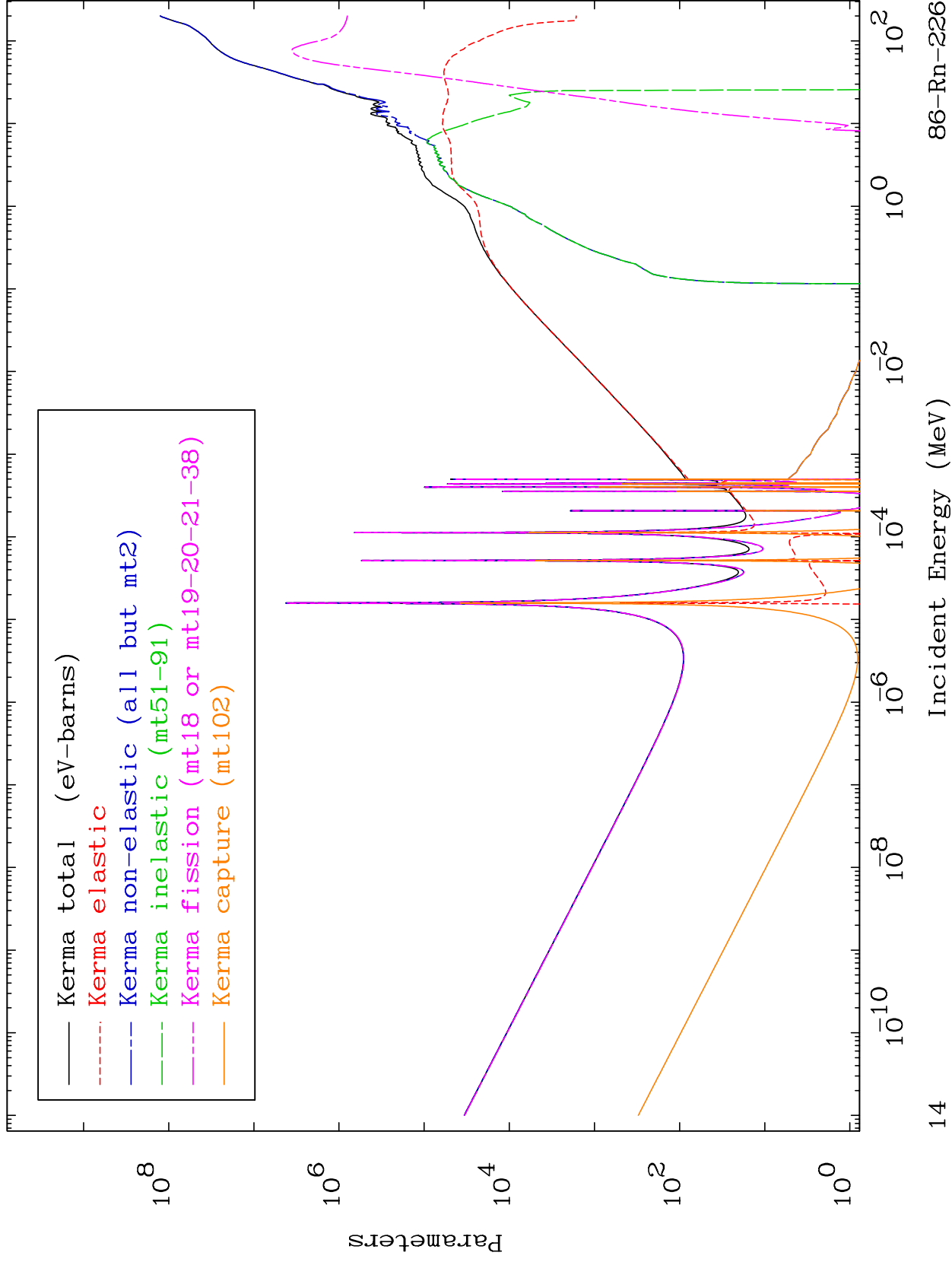
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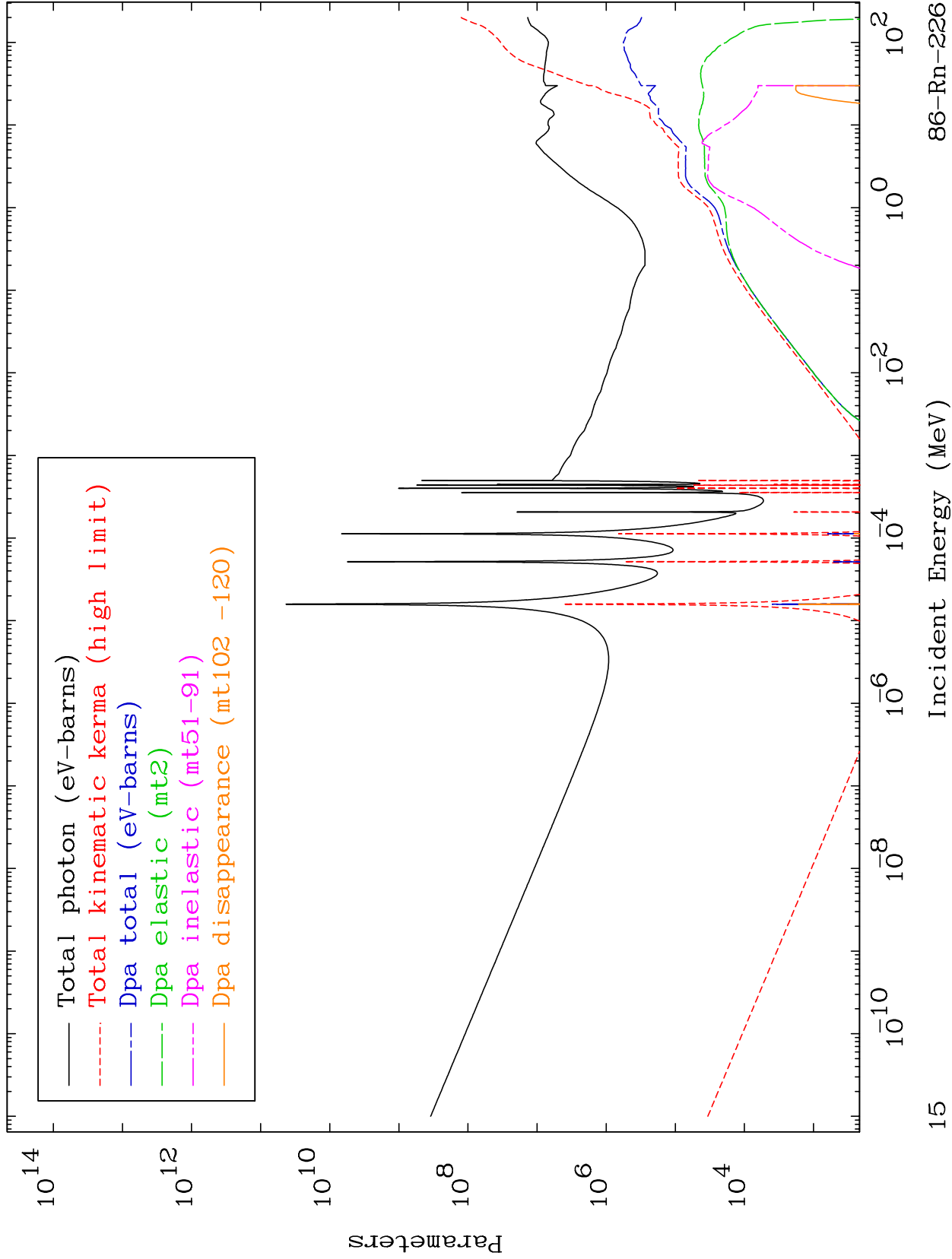
MAT 8670

(n, α) Levels
293 Kelvin Cross Sections

86-Rn-226



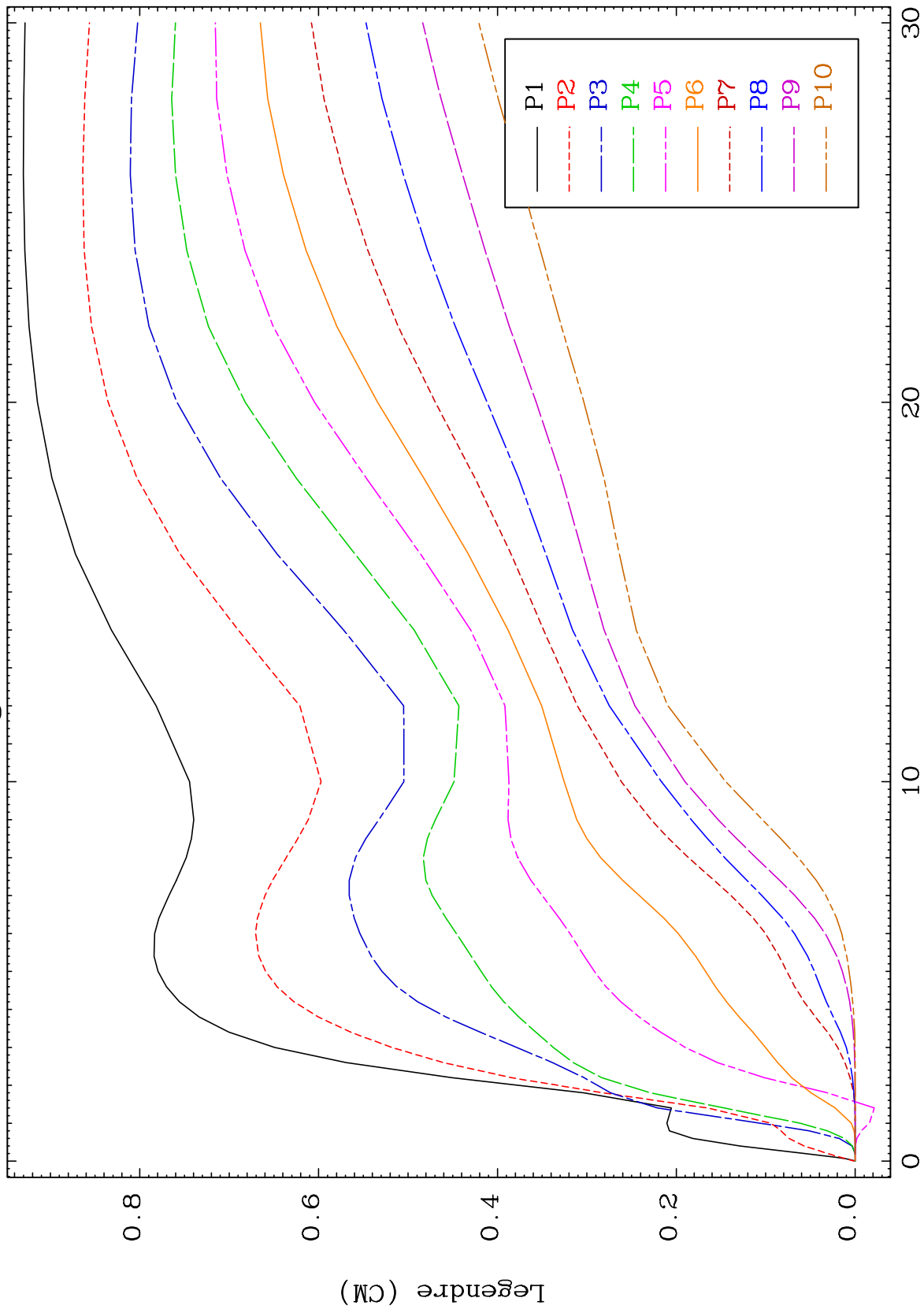




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Elastic Legendre Coefficients

86-Rn-226



16

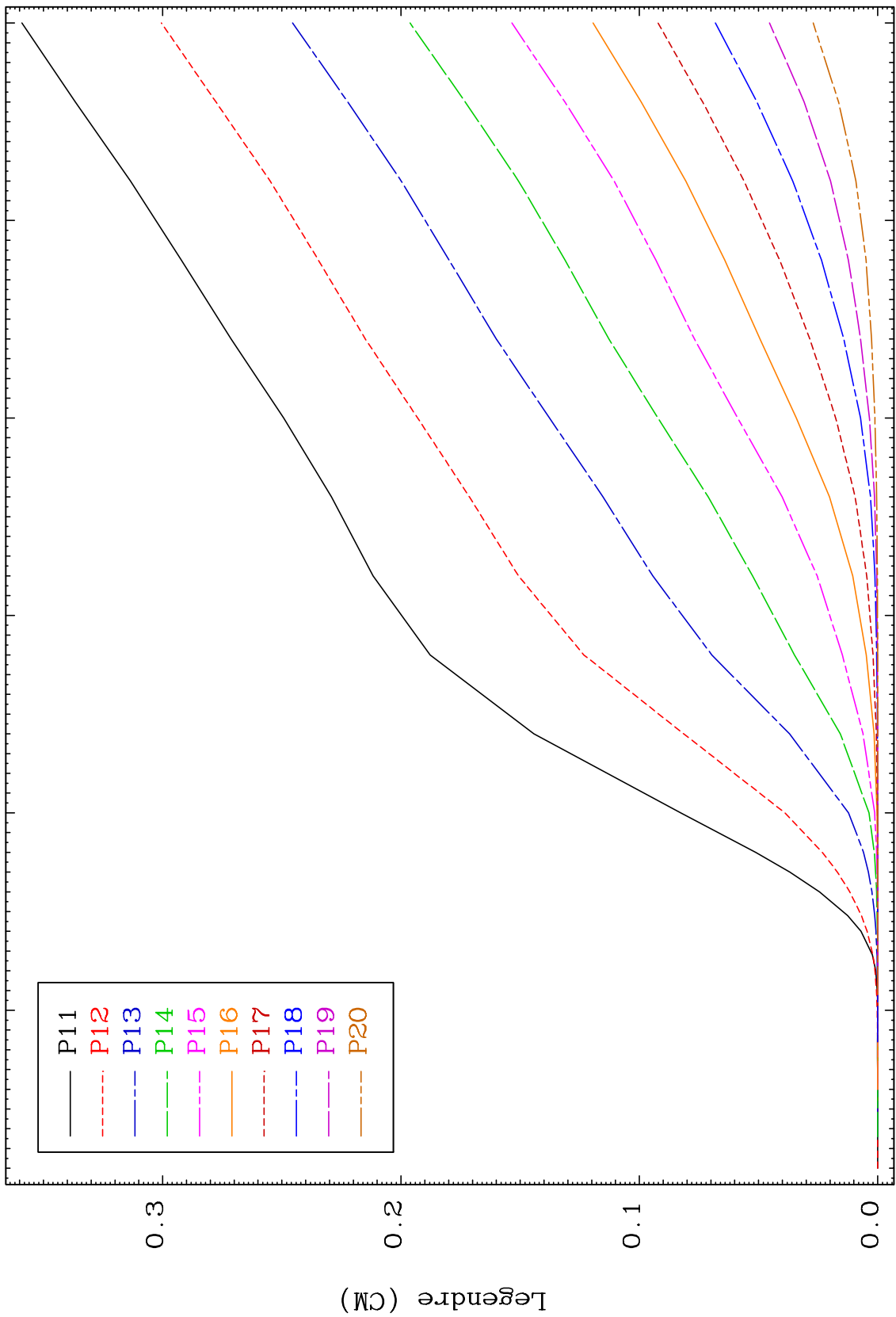
Incident Energy (MeV)

86-Rn-226

MAT 8670

Elastic Legendre Coefficients

86-Rn-226



17

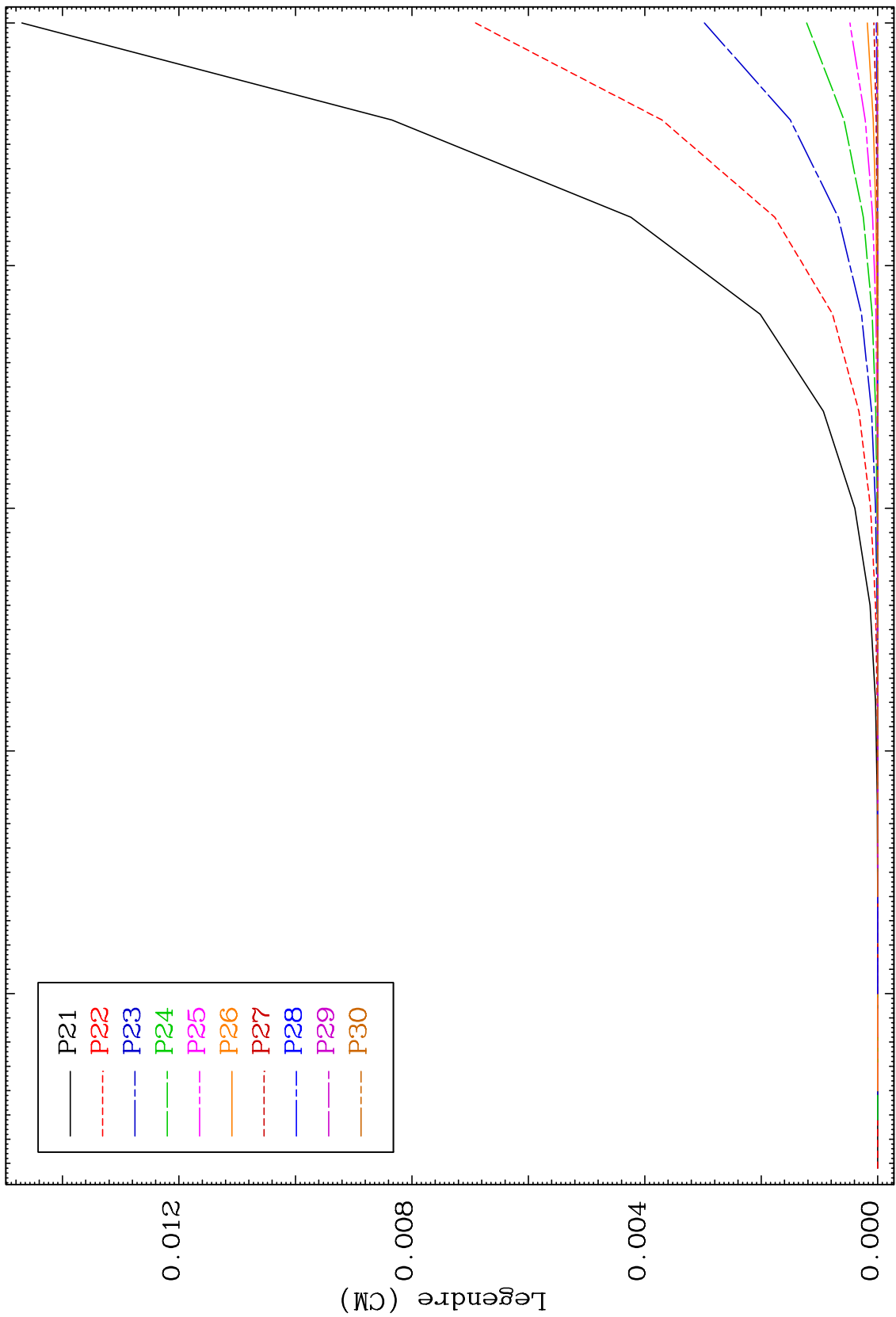
Incident Energy (MeV)

86-Rn-226

MAT 8670

Elastic Legendre Coefficients

86-Rn-226



18

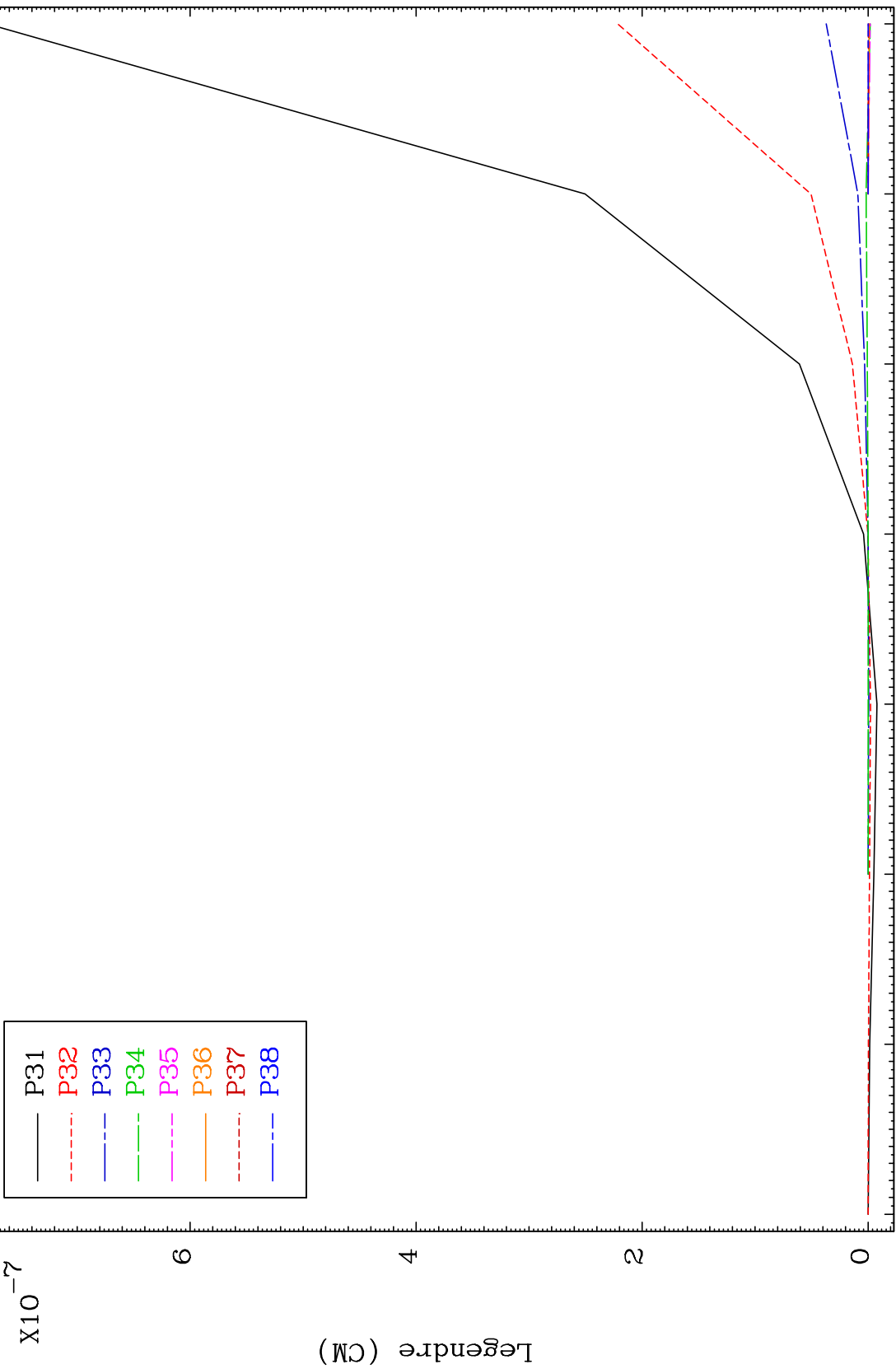
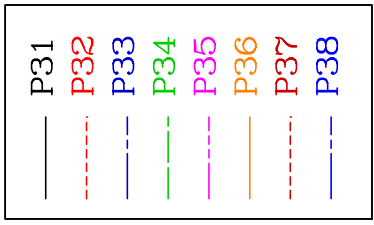
Incident Energy (MeV)

86-Rn-226

MAT 8670

Elastic Legendre Coefficients

86-Rn-226

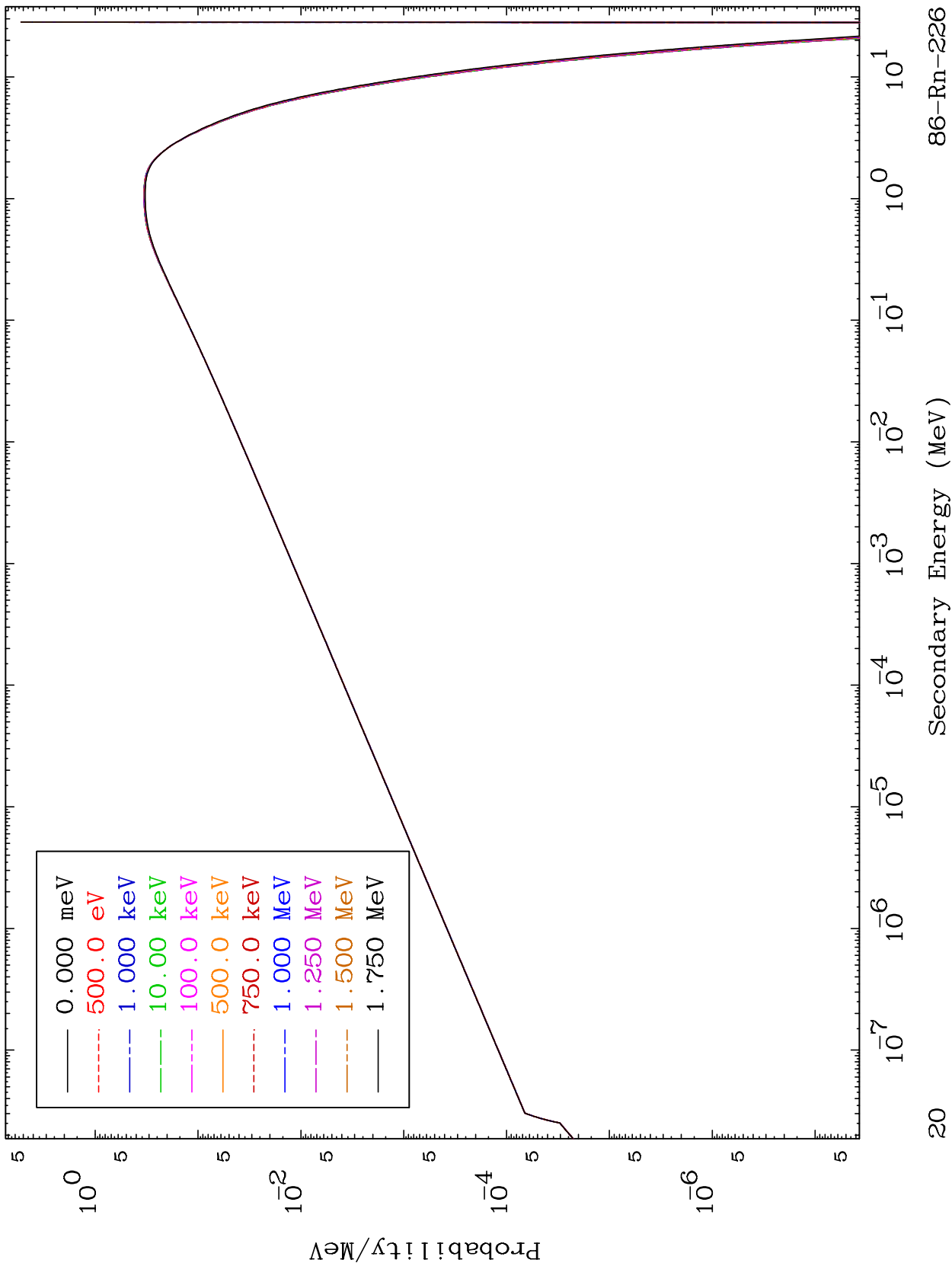


19

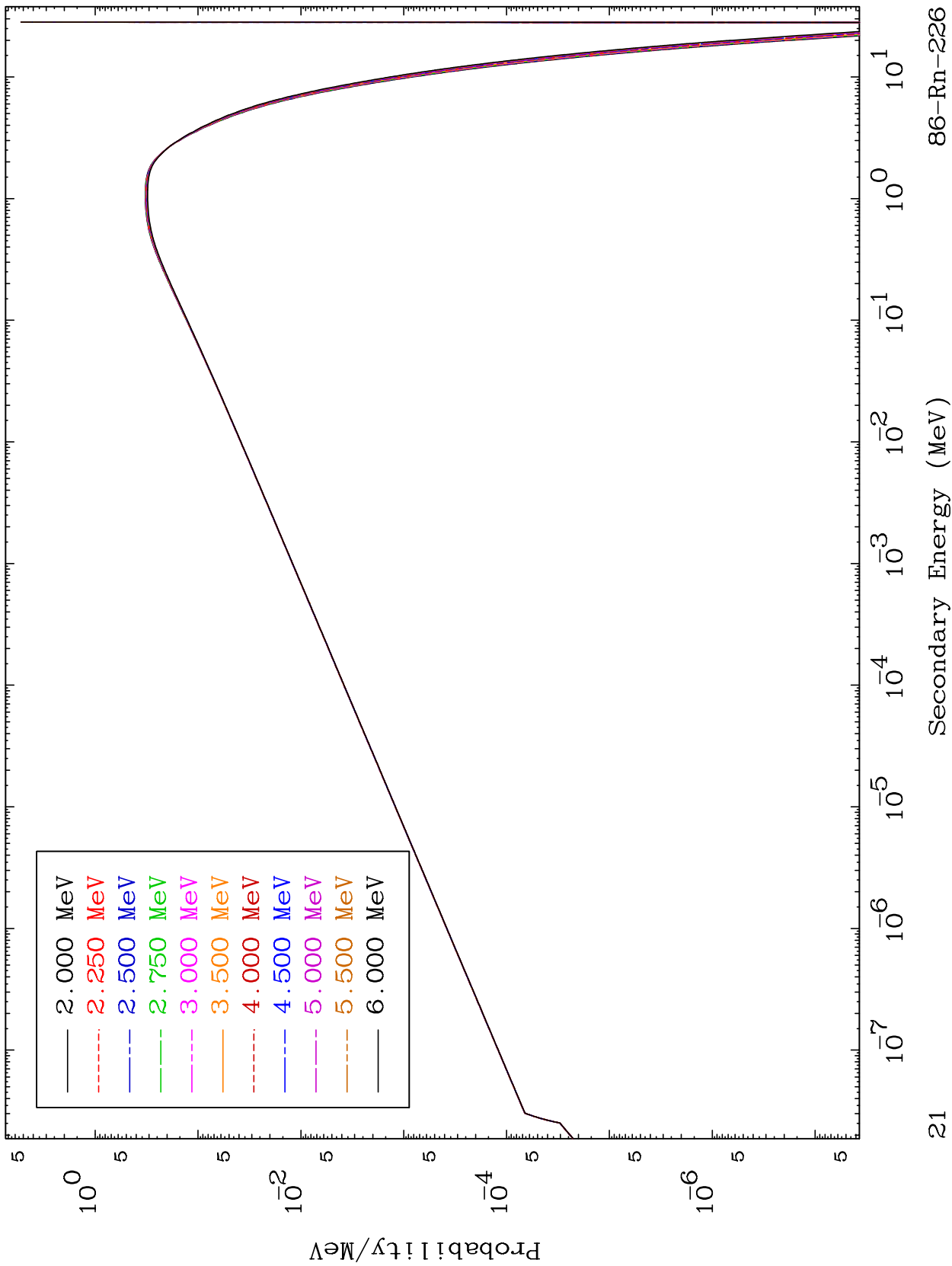
Incident Energy (MeV)

86-Rn-226

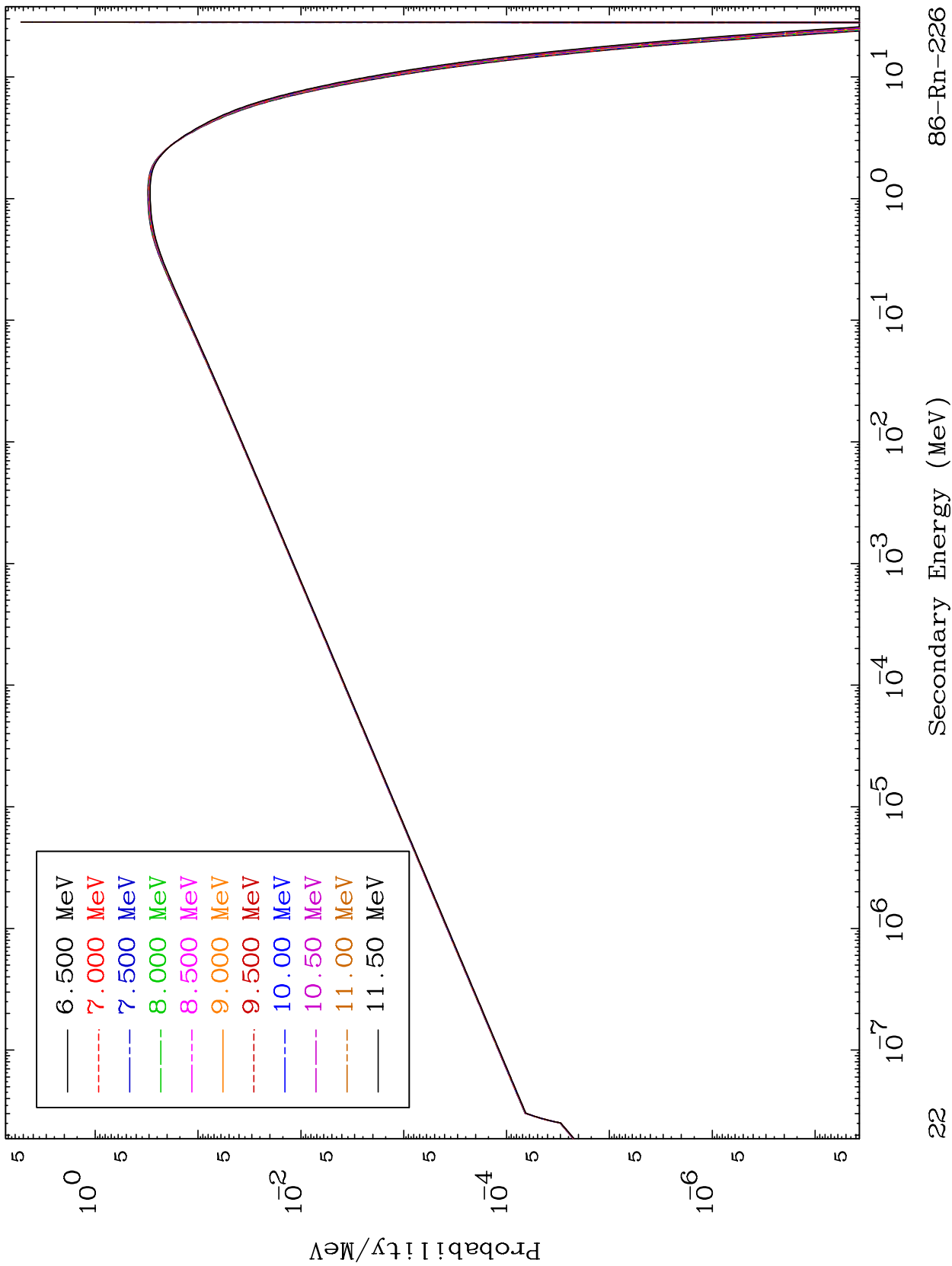
Fission Energy Distribution

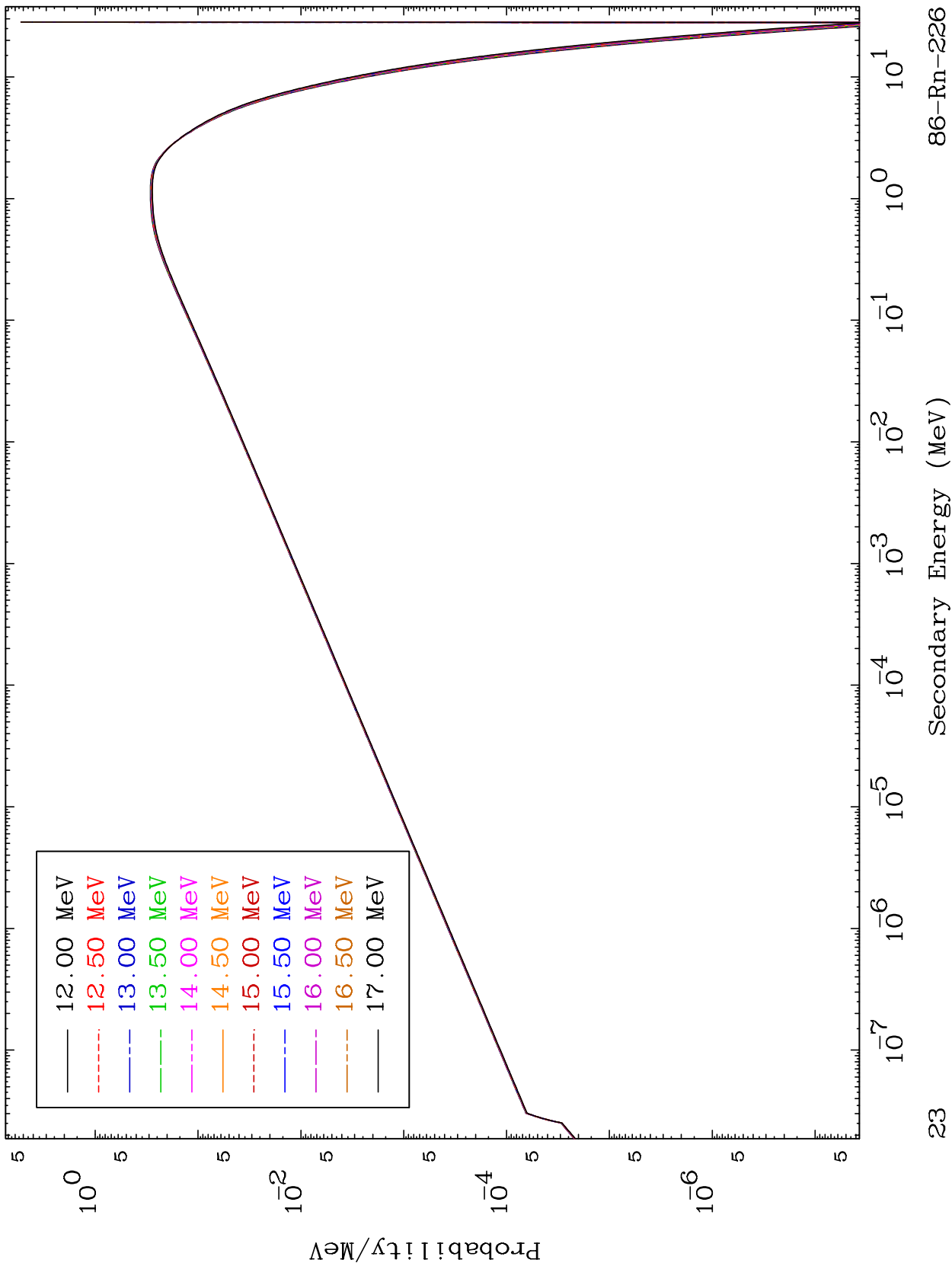


Fission
Energy Distribution



Fission
Energy Distribution





Fission
Energy Distribution

