

Program EVALPLOT  
(Version 2015-2)

by

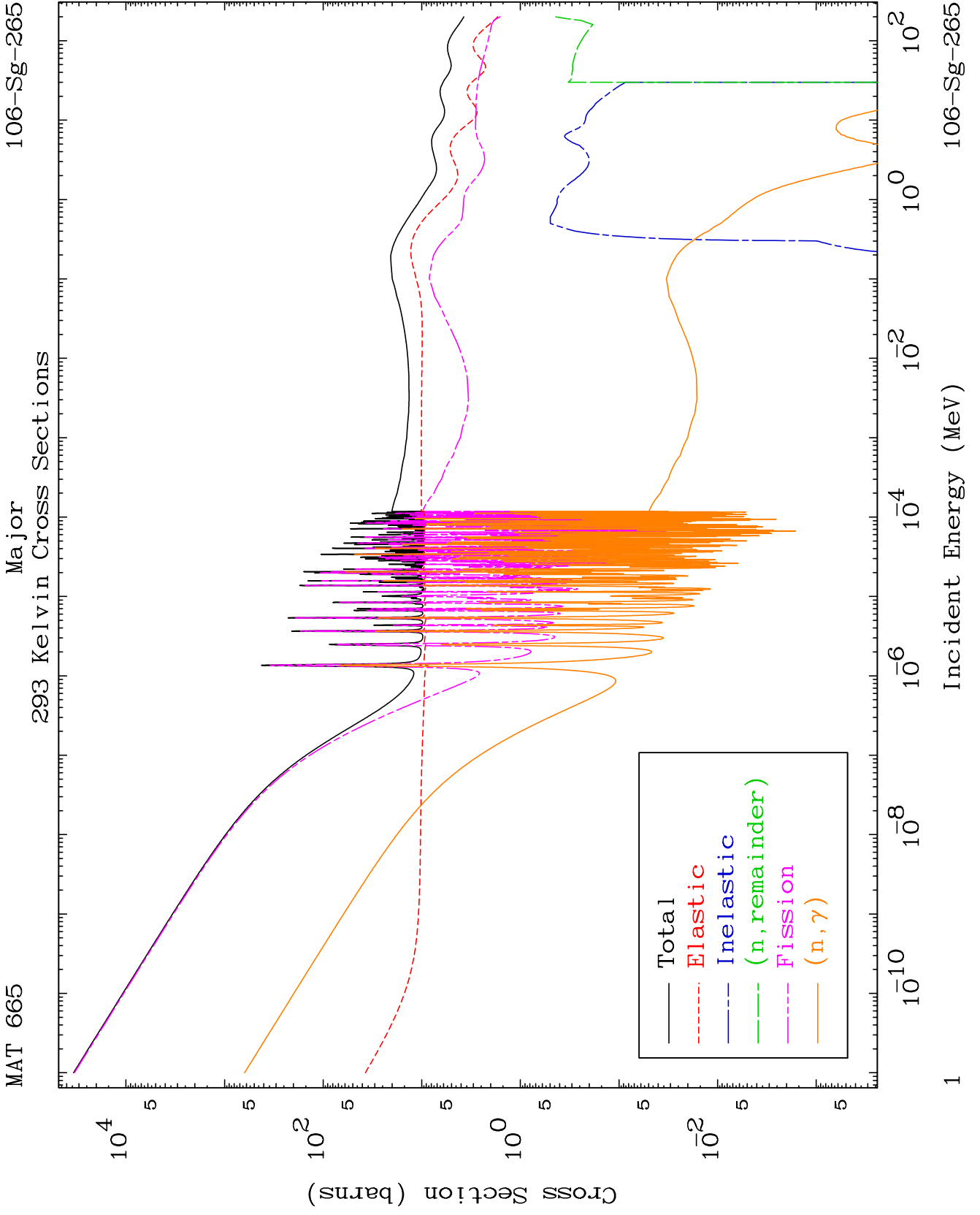
Dermott E. Cullen  
(Present Contact Information)

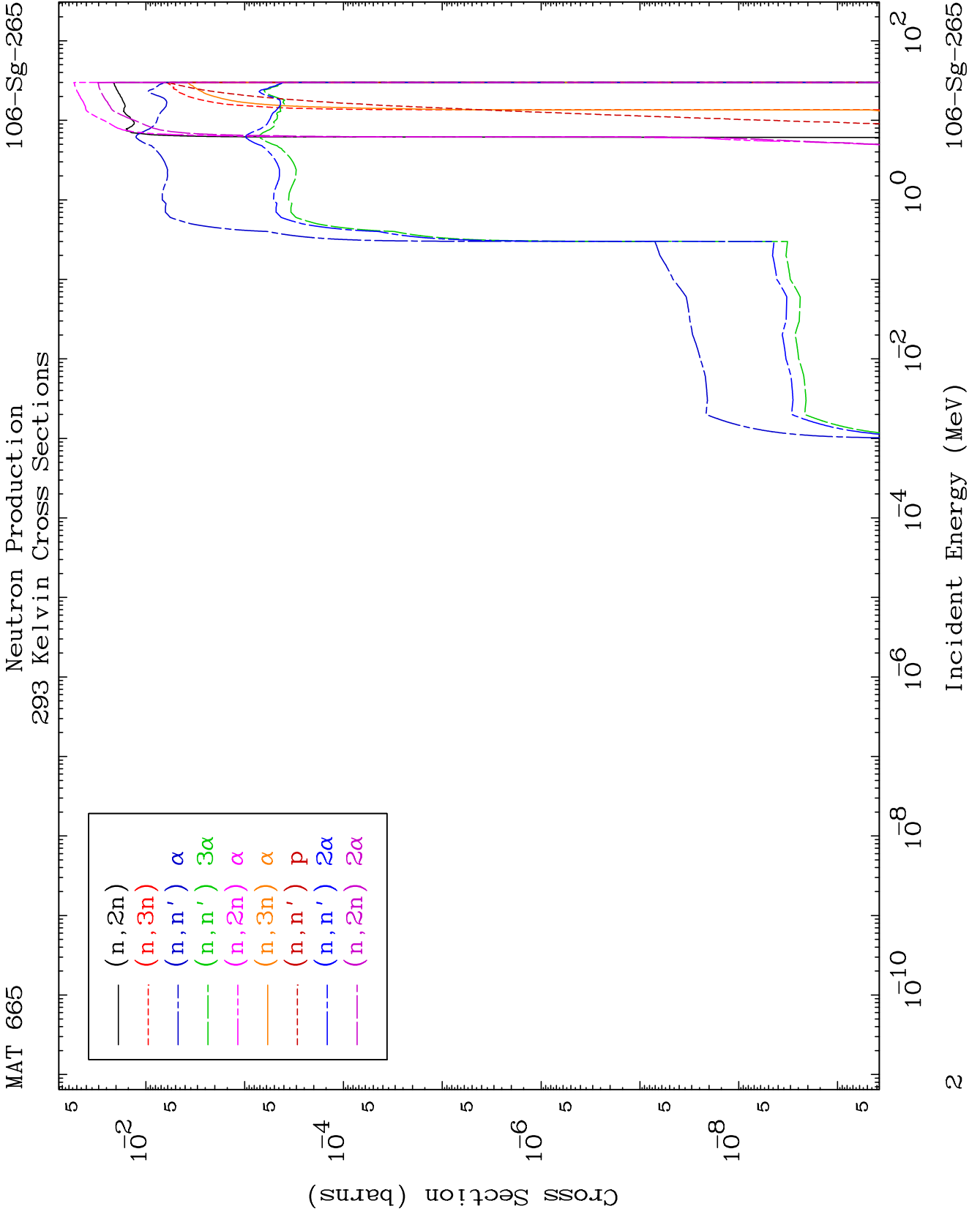
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Livermore, CA 94550  
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E.Mail:redcullen1@comcast.net  
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Press Mouse Button to Start

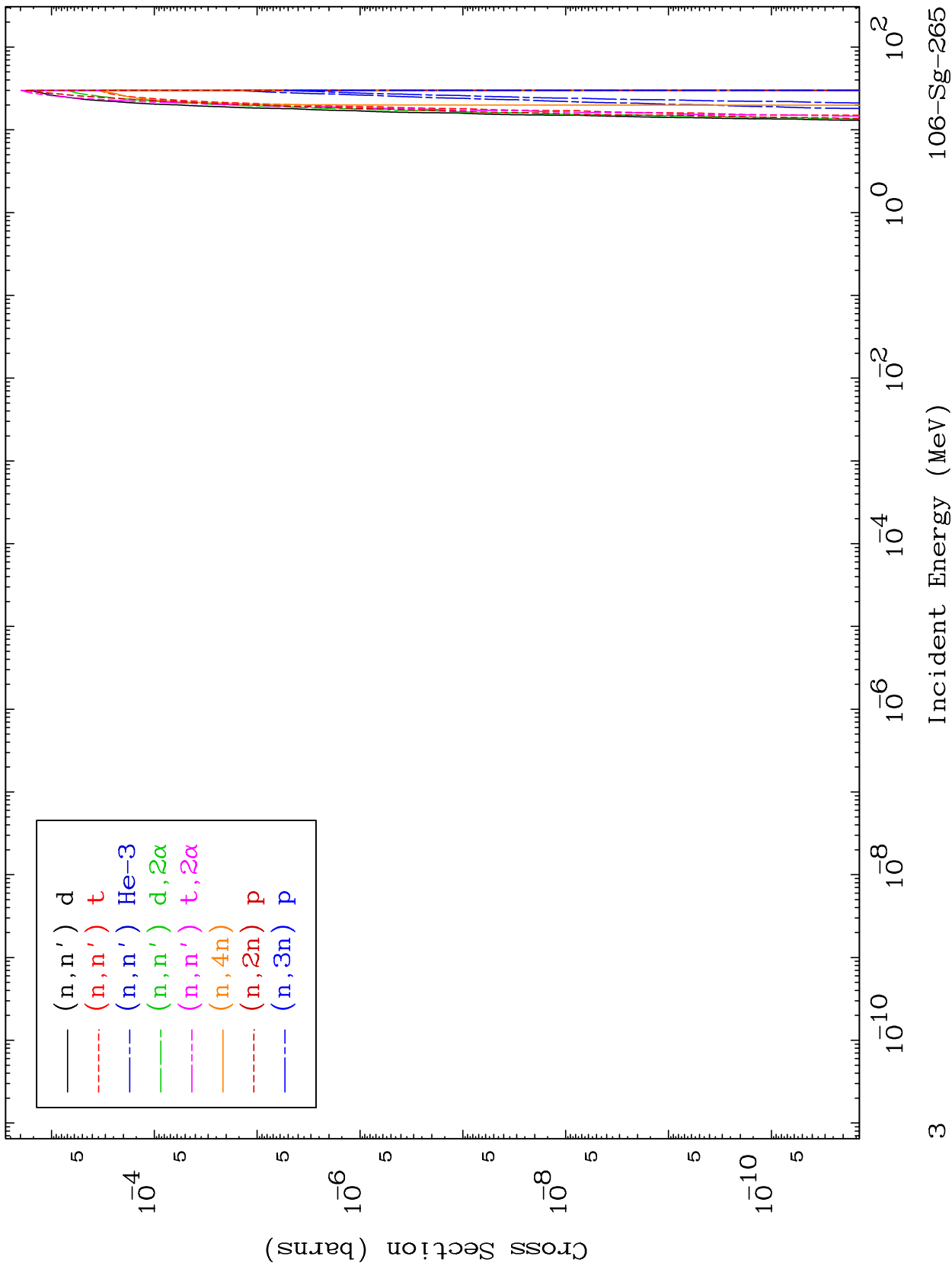


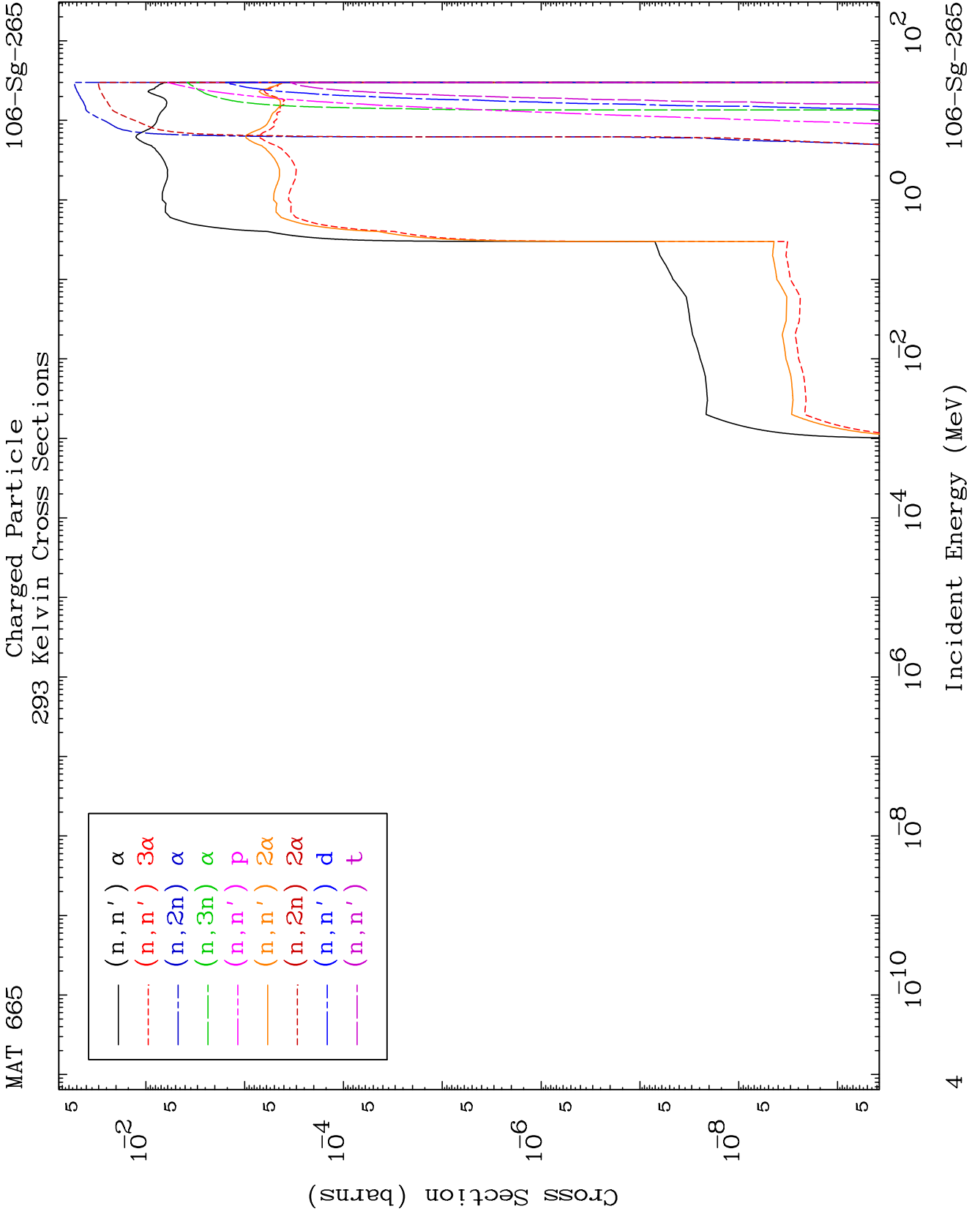


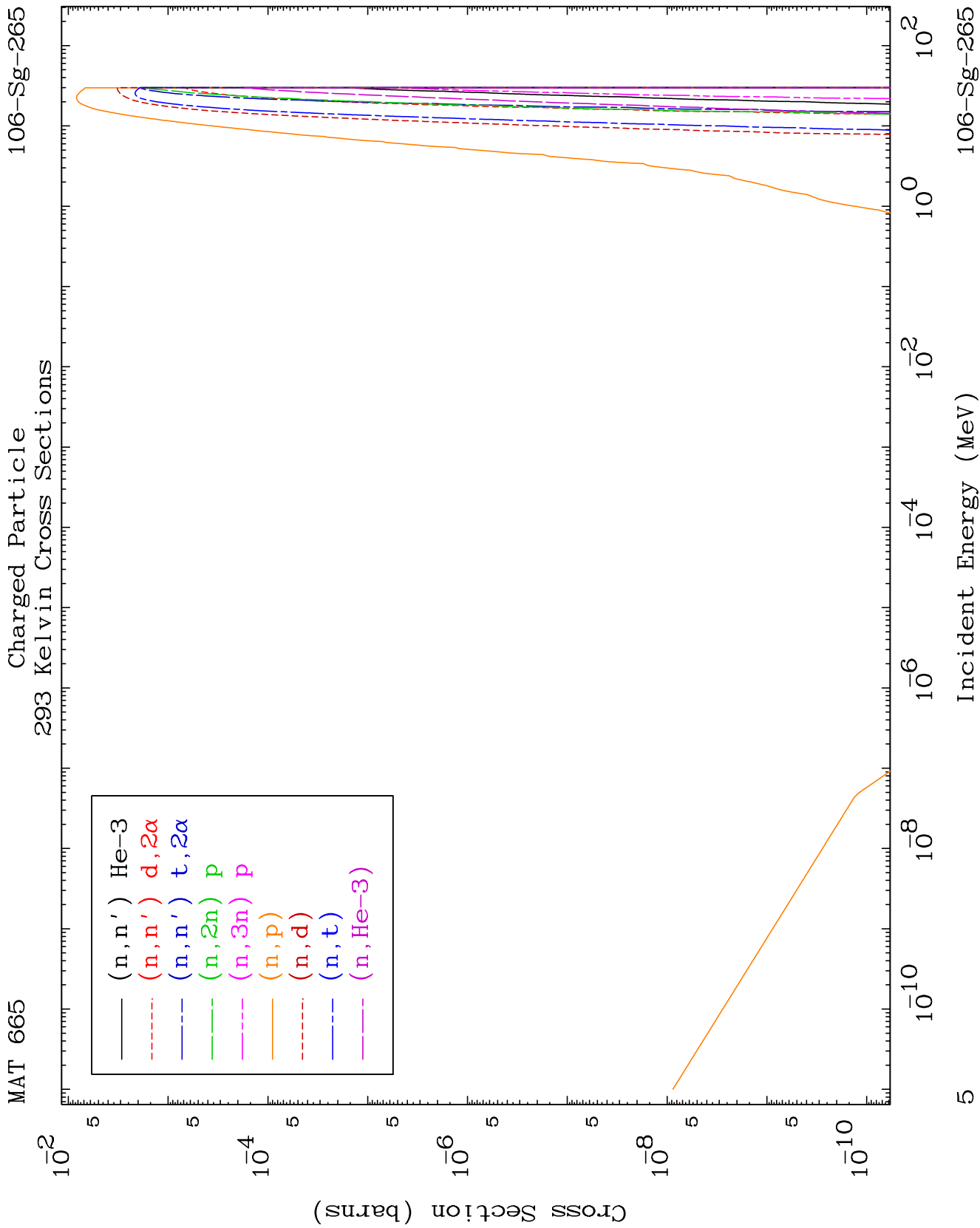
MAT 665

Neutron Production  
293 Kelvin Cross Sections

106-Sg-265



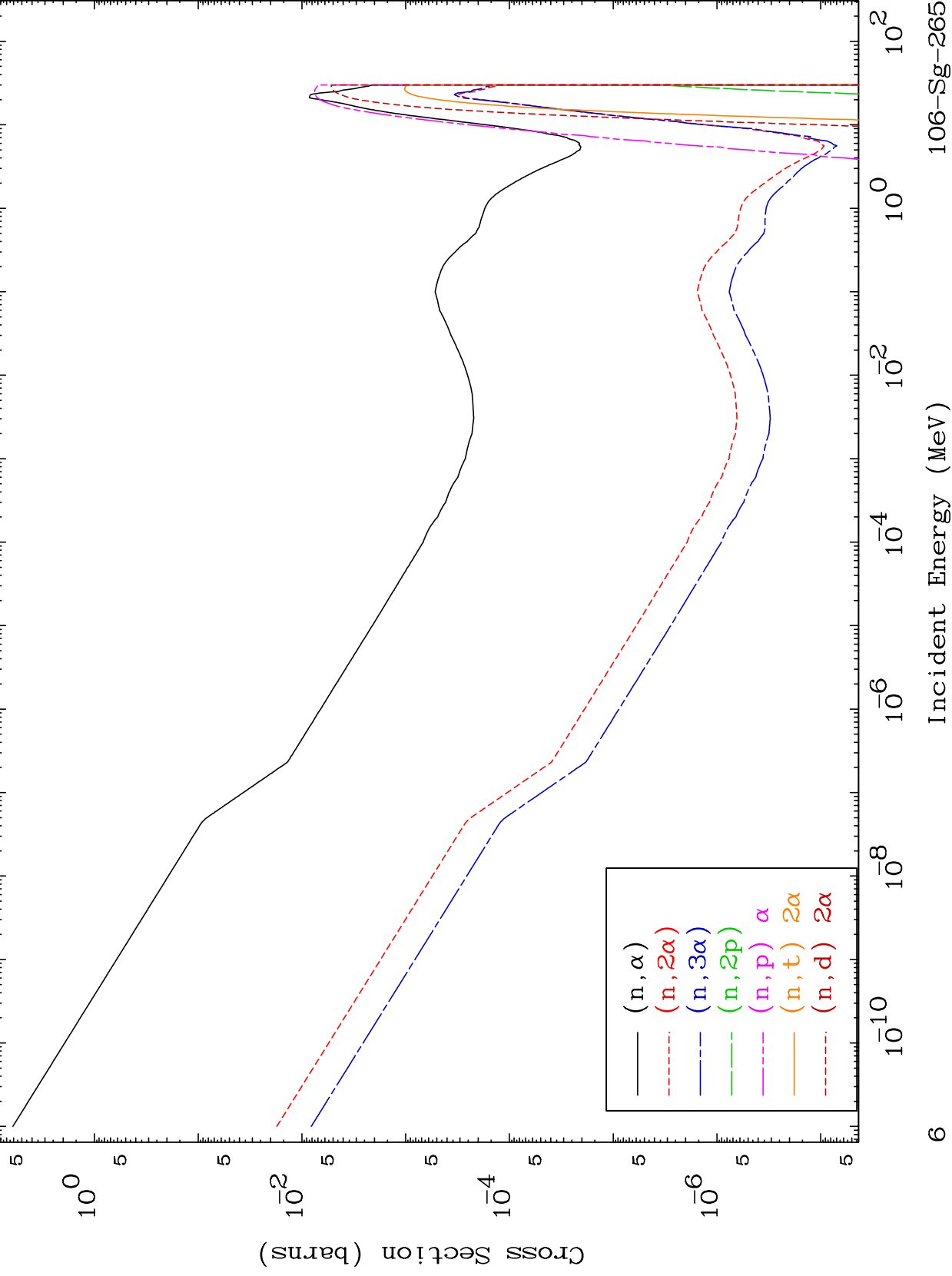


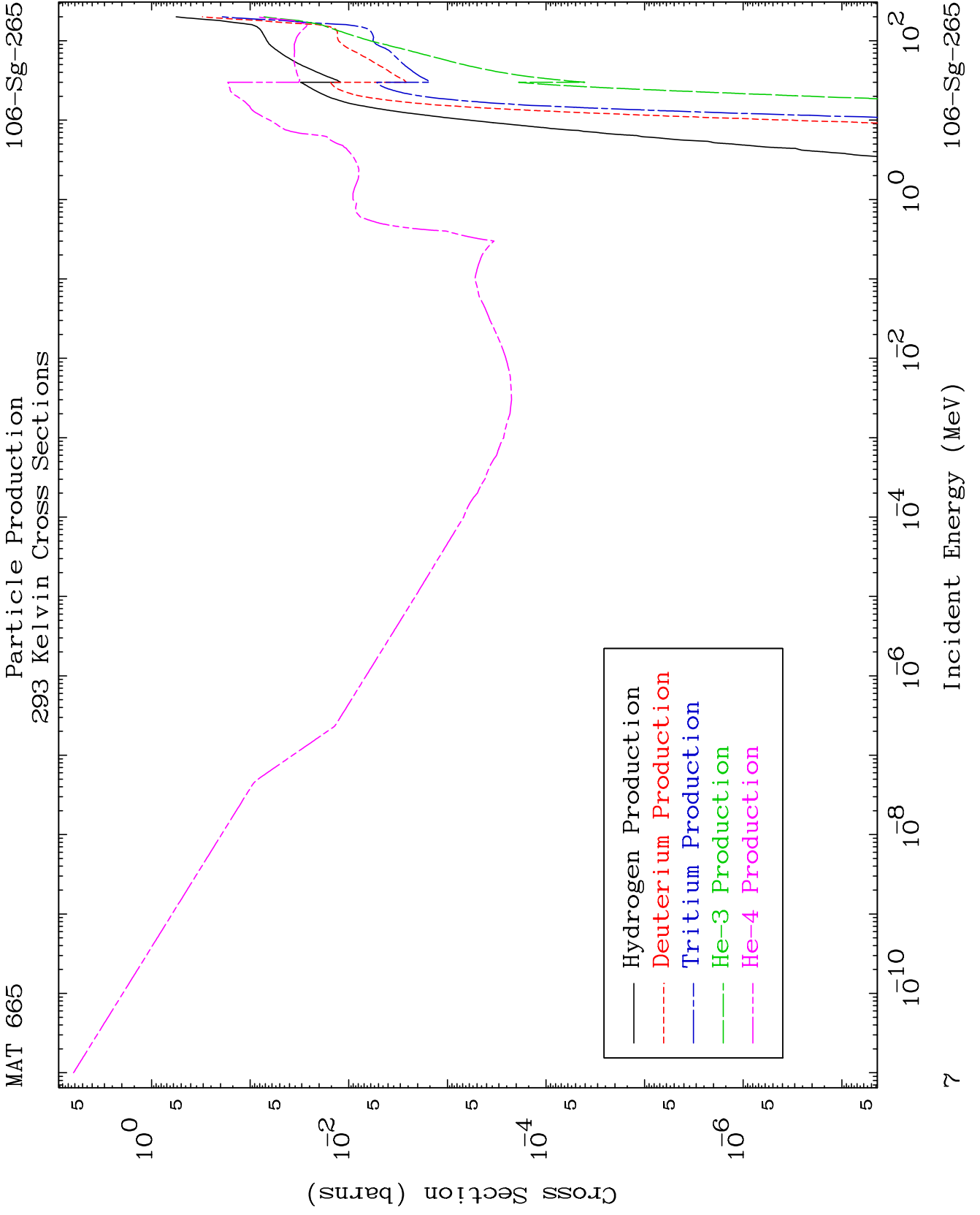


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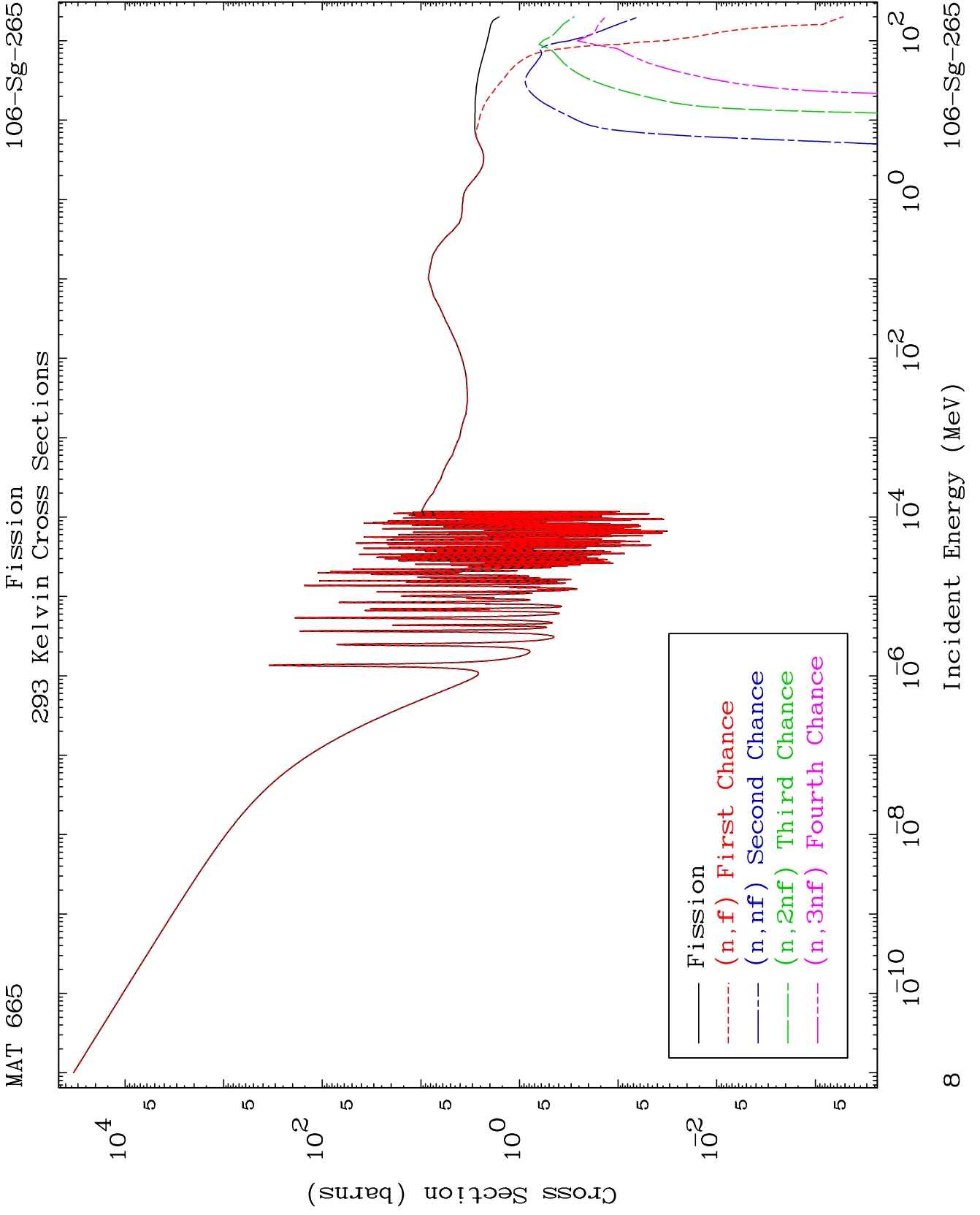
Charged Particle  
293 Kelvin Cross Sections

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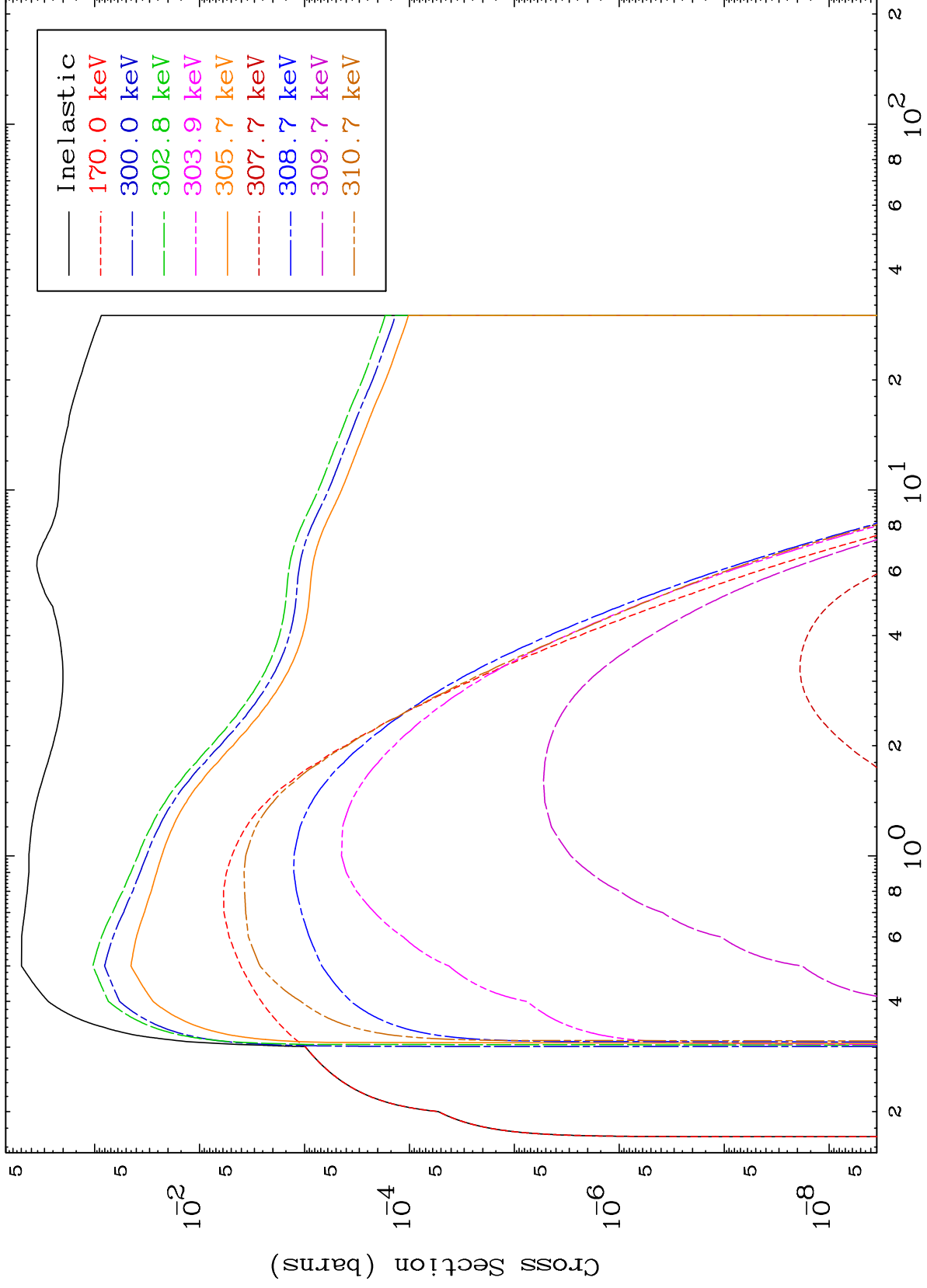


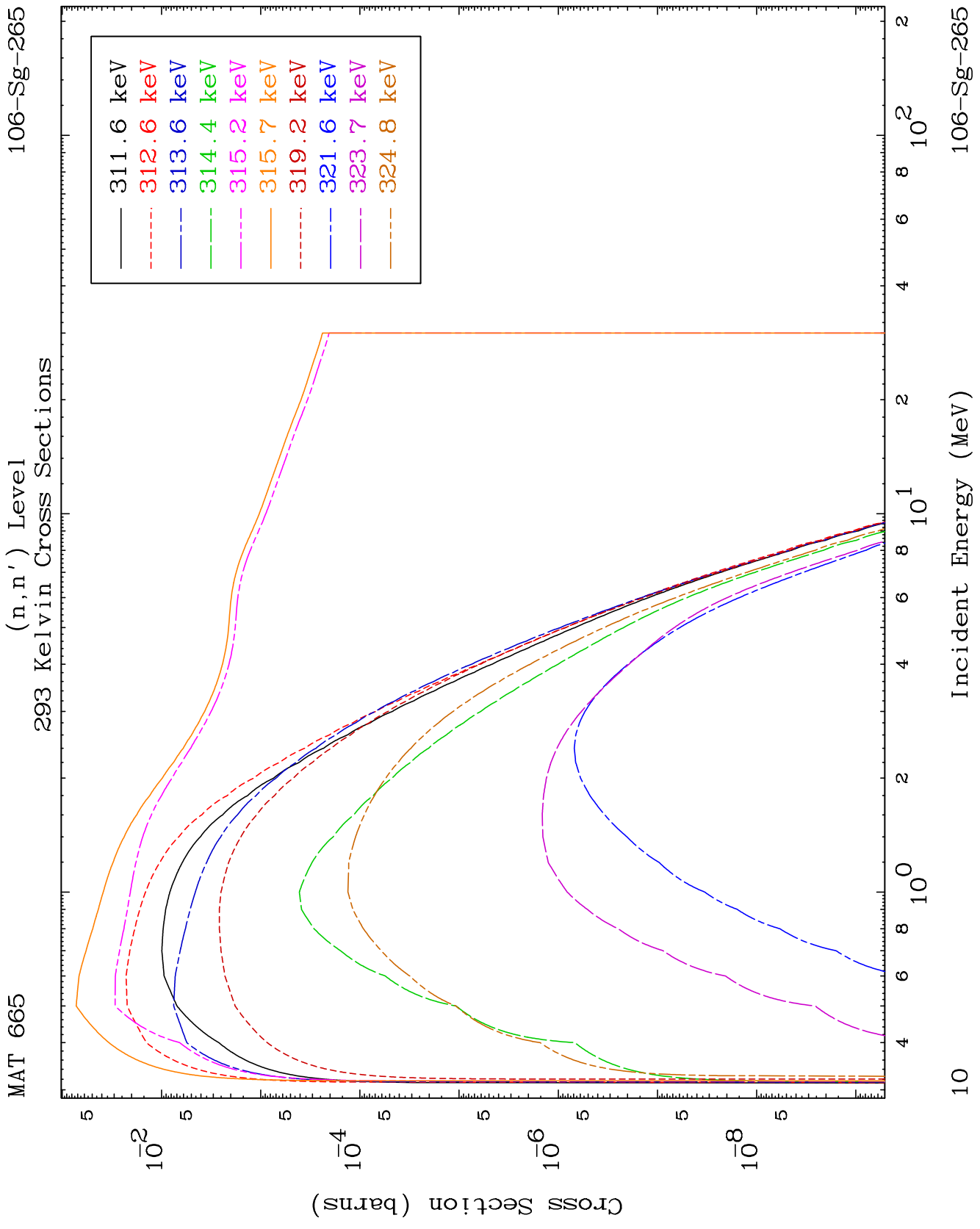


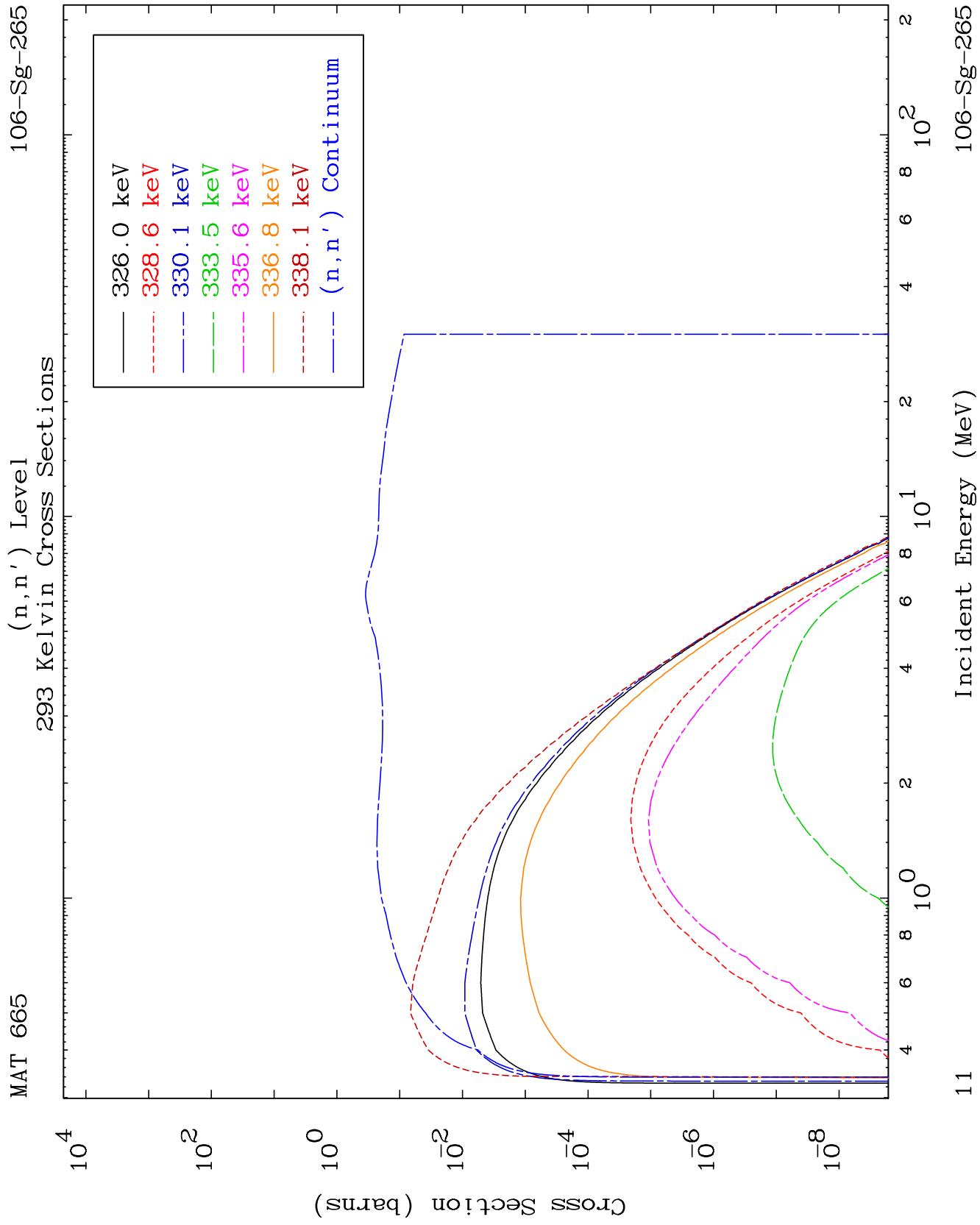
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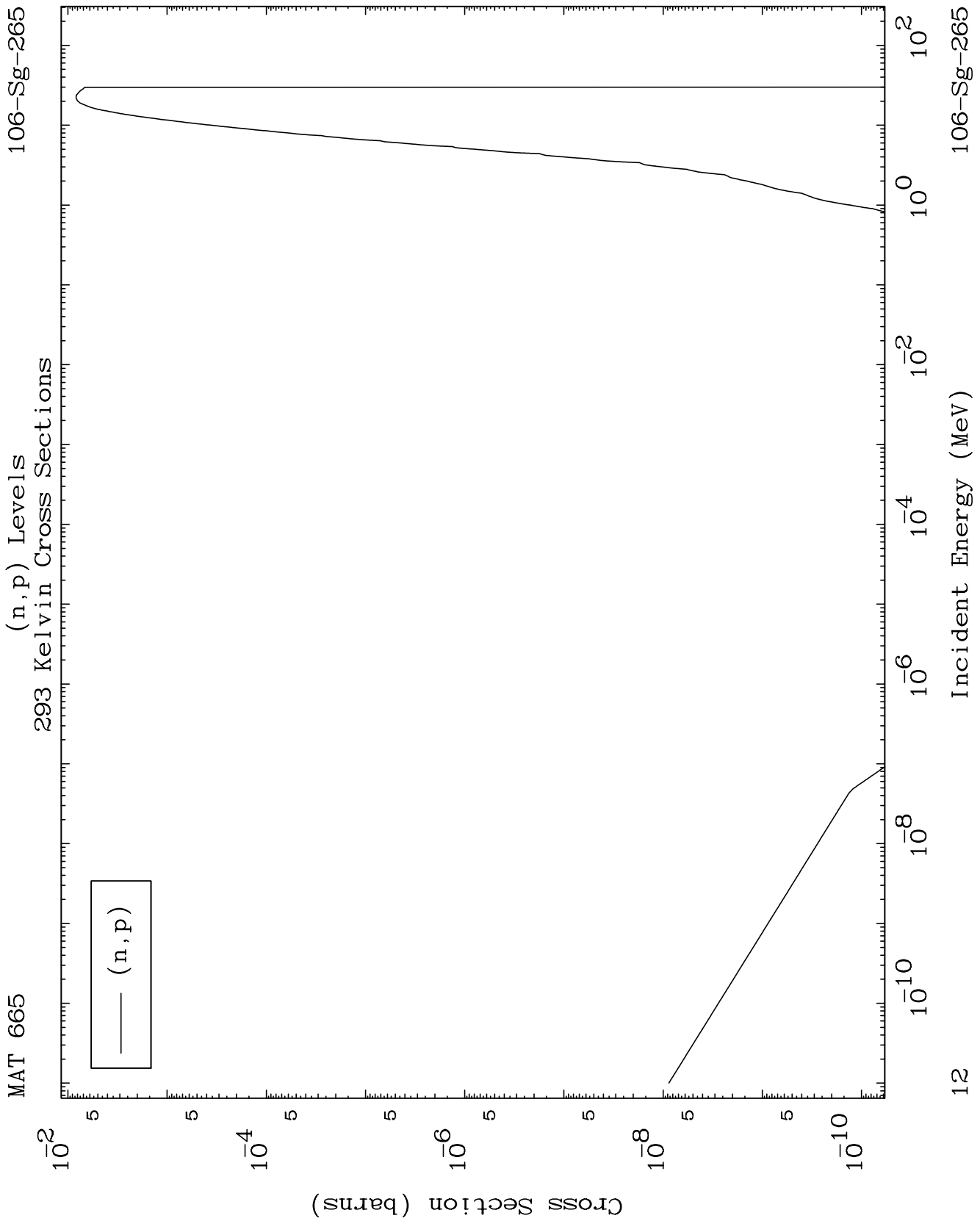
(n,n') Level  
293 Kelvin Cross Sections

106-Sg-265





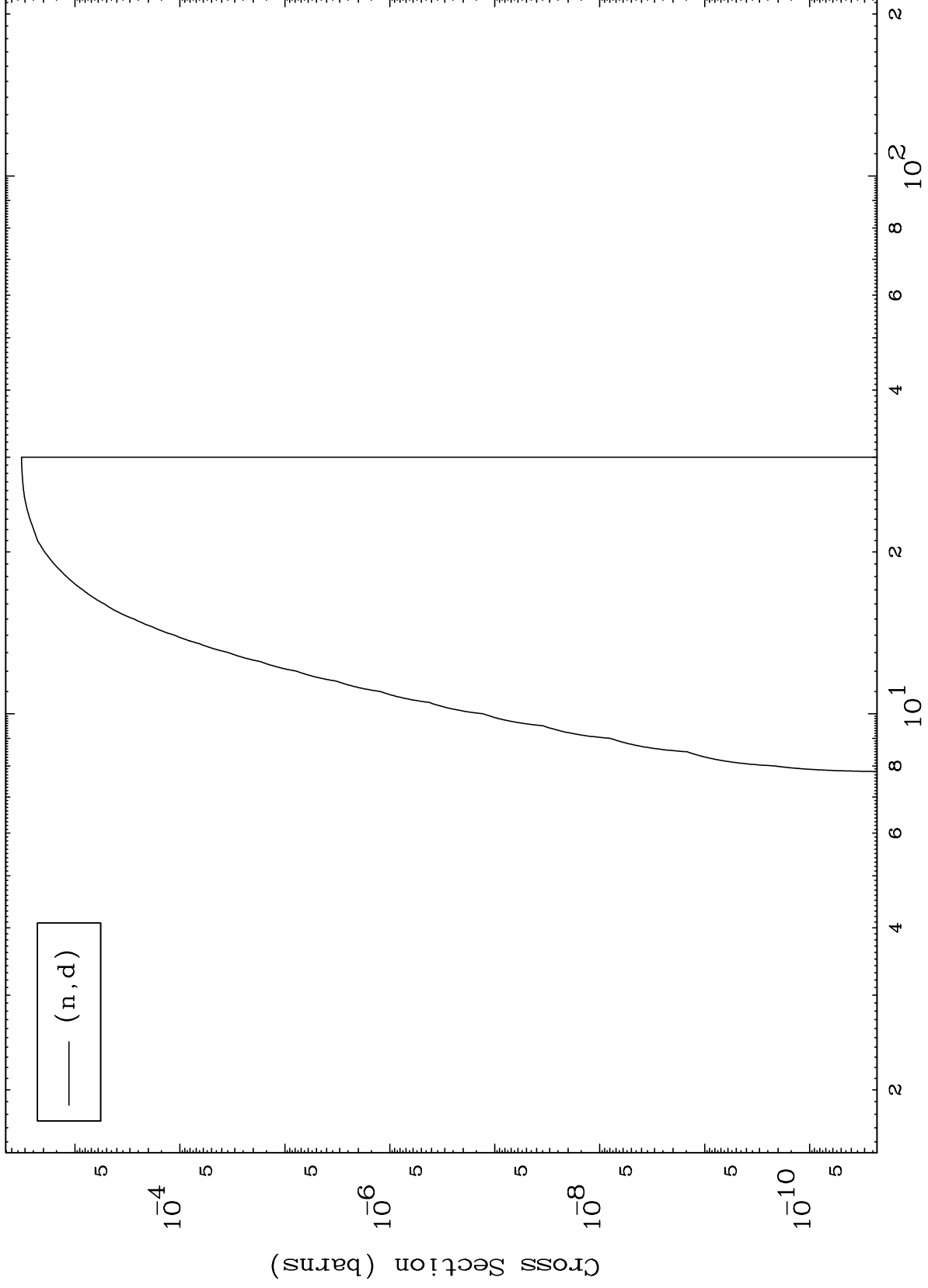




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(n,d) Levels  
293 Kelvin Cross Sections

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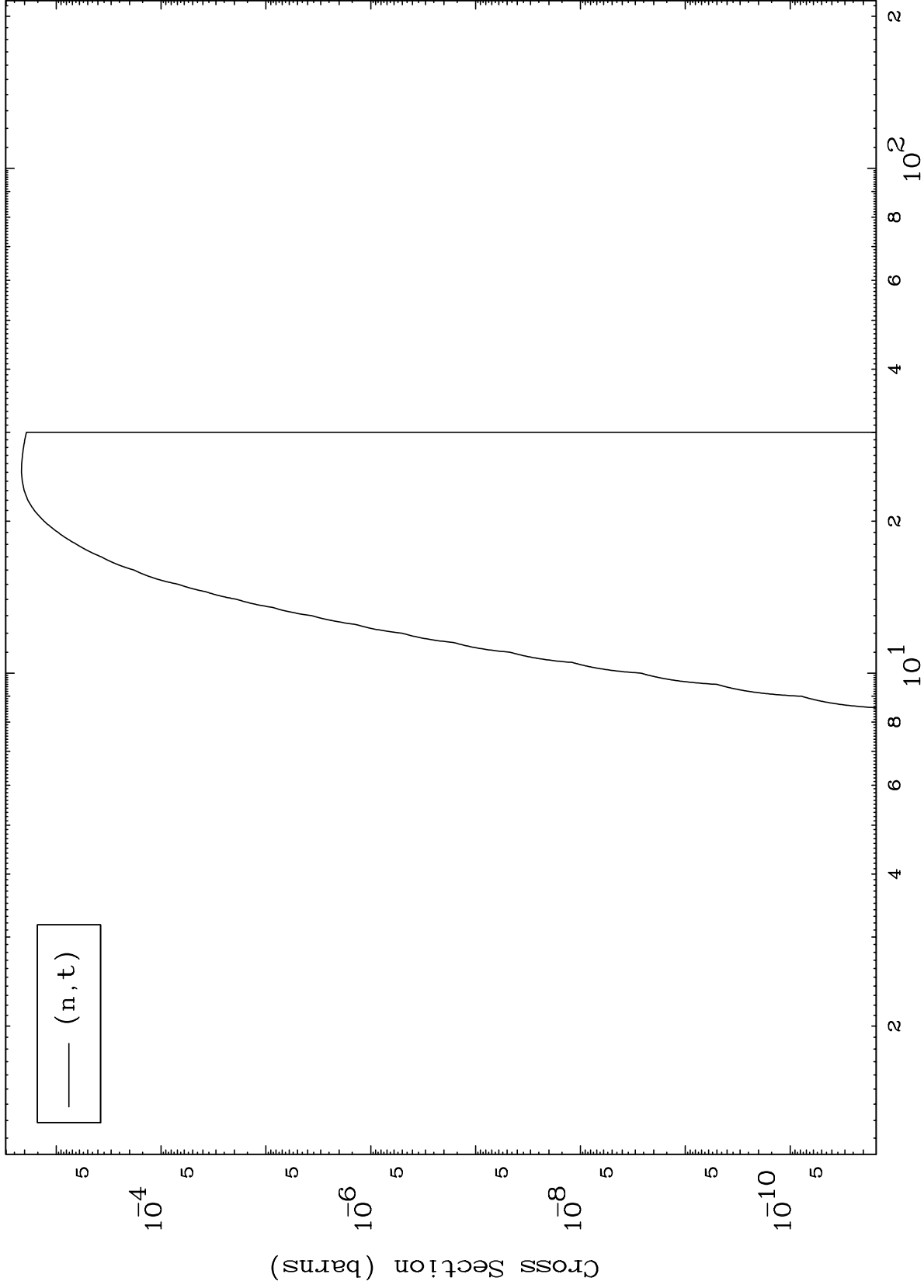
Incident Energy (MeV)

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(n,t) Levels  
293 Kelvin Cross Sections

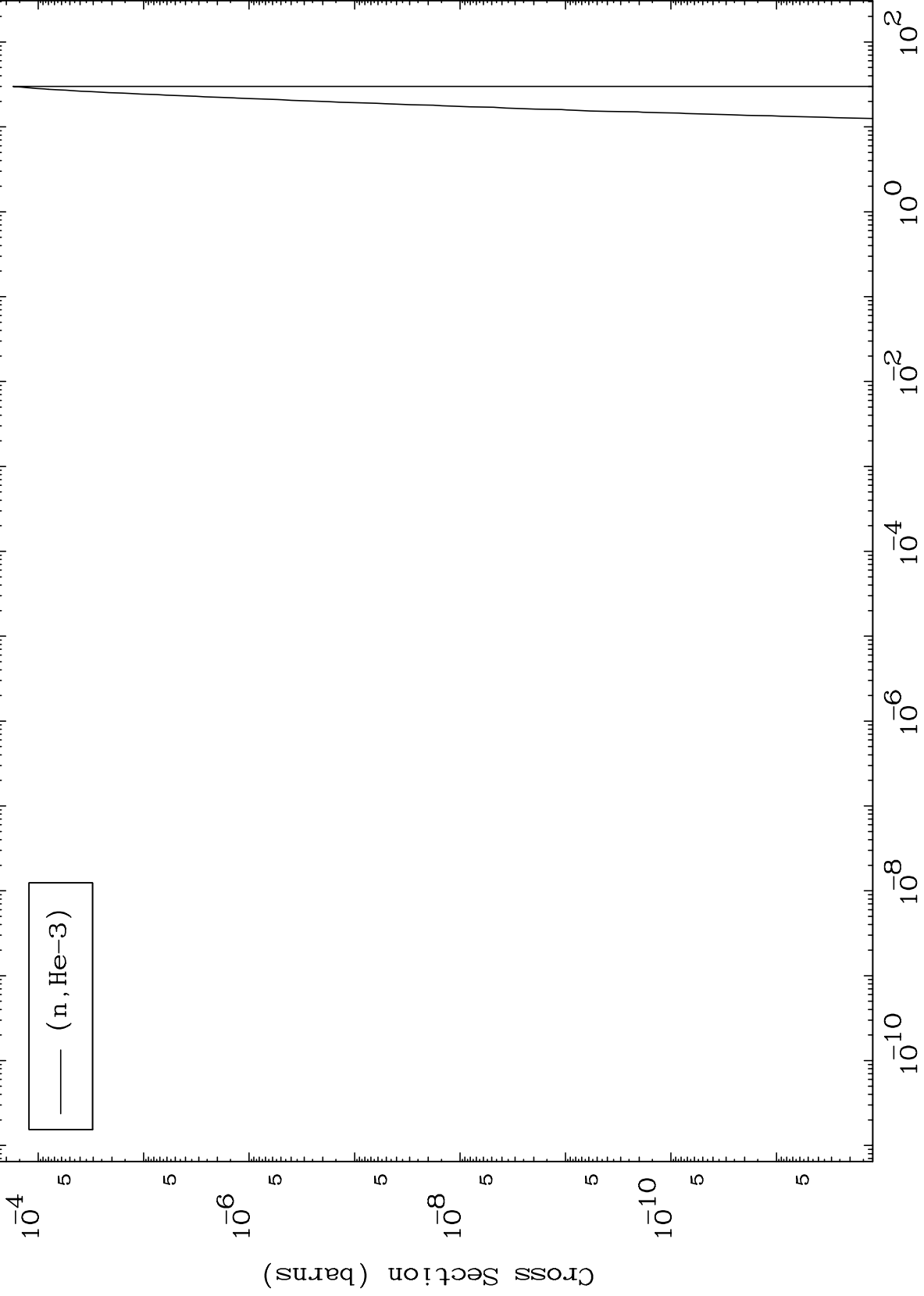
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(n,He3) Levels  
293 Kelvin Cross Sections

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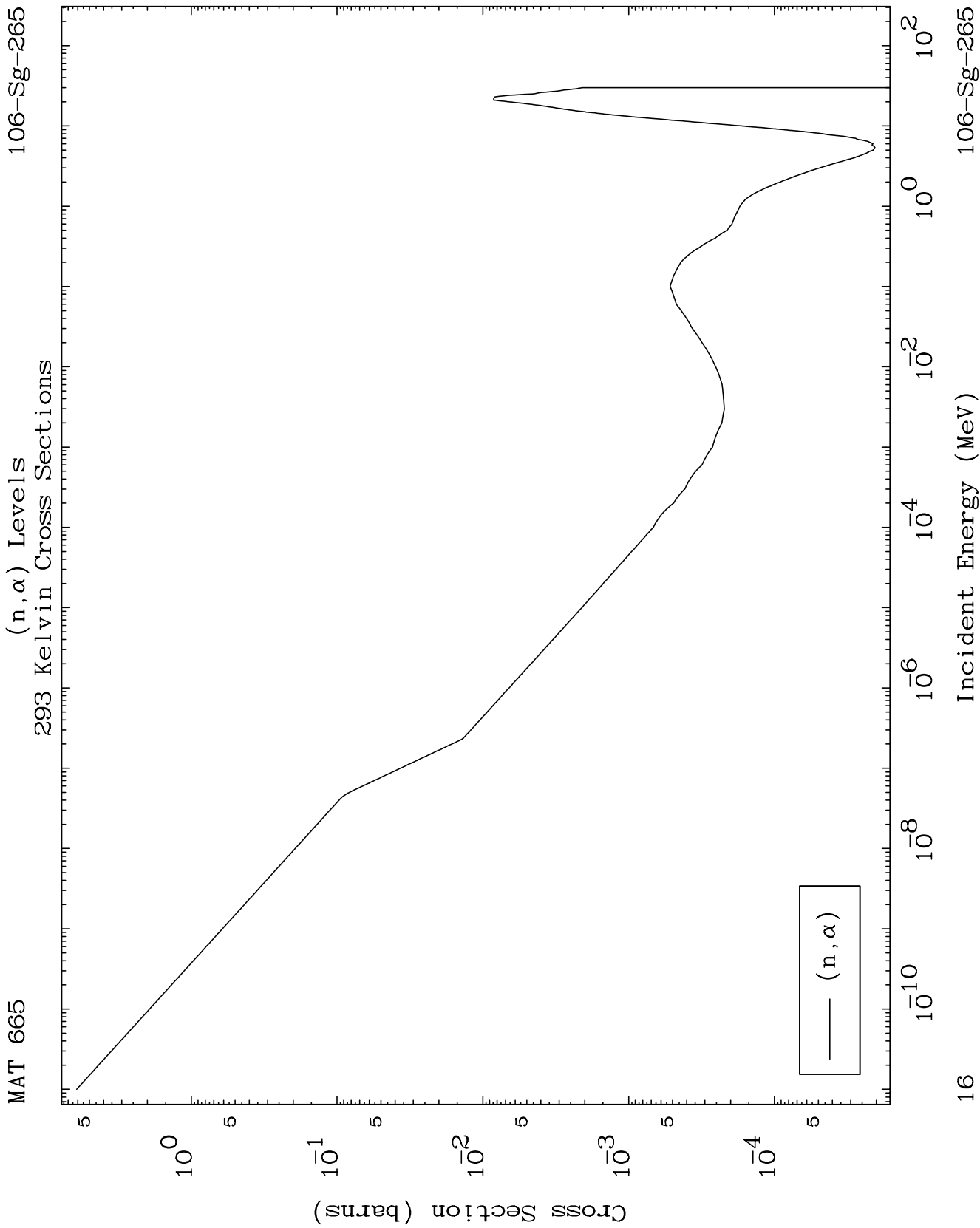


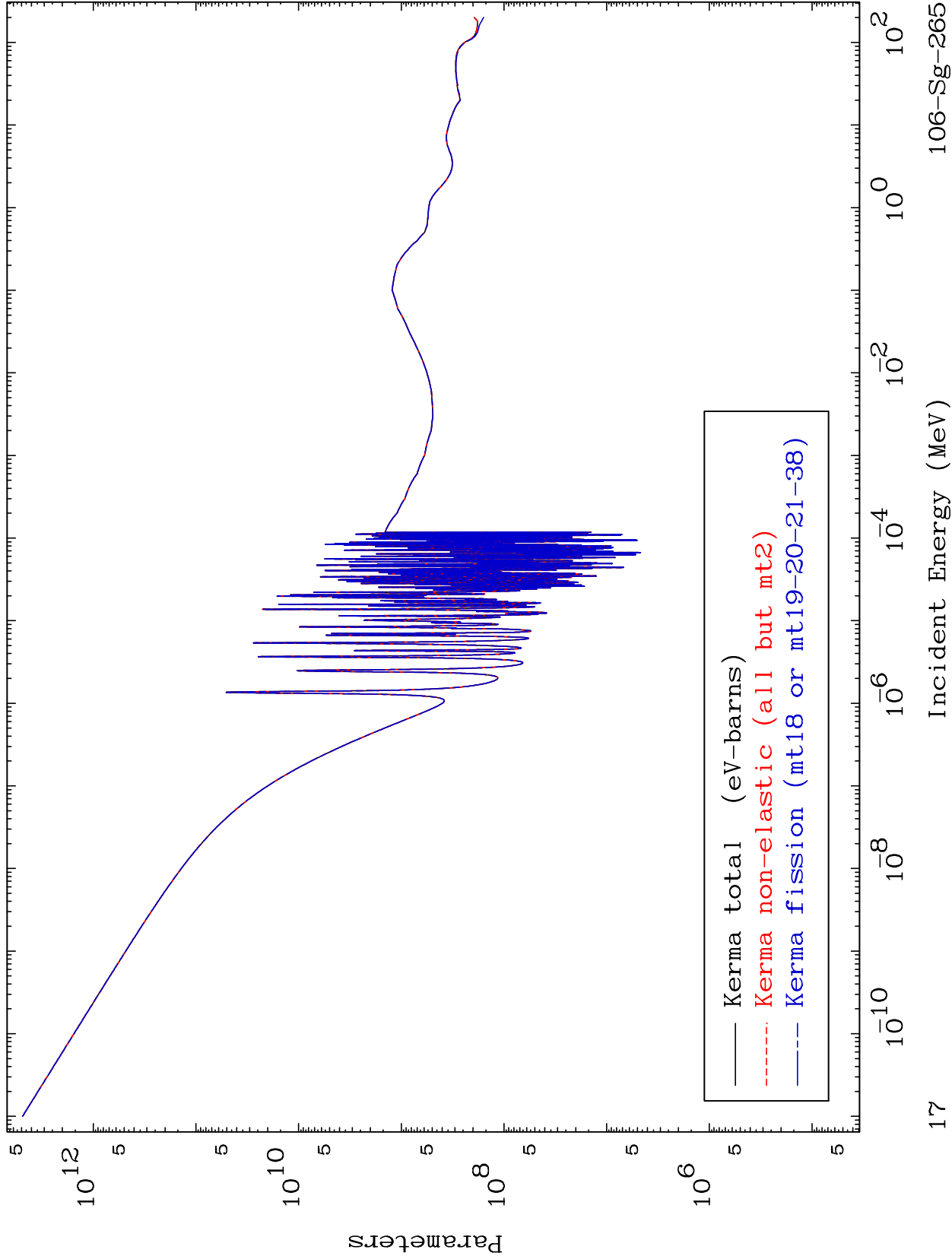
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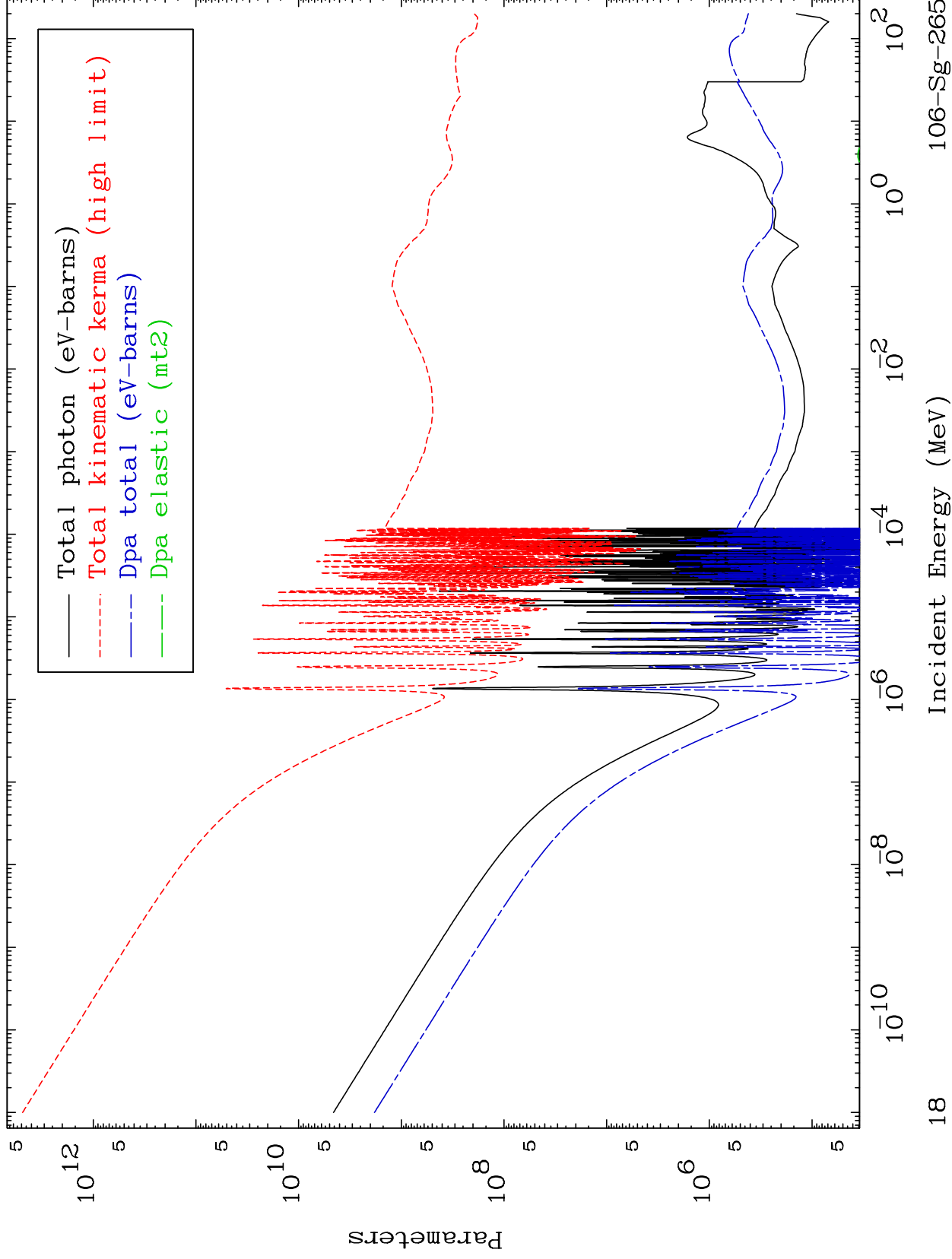
Incident Energy (MeV)

106-Sg-265

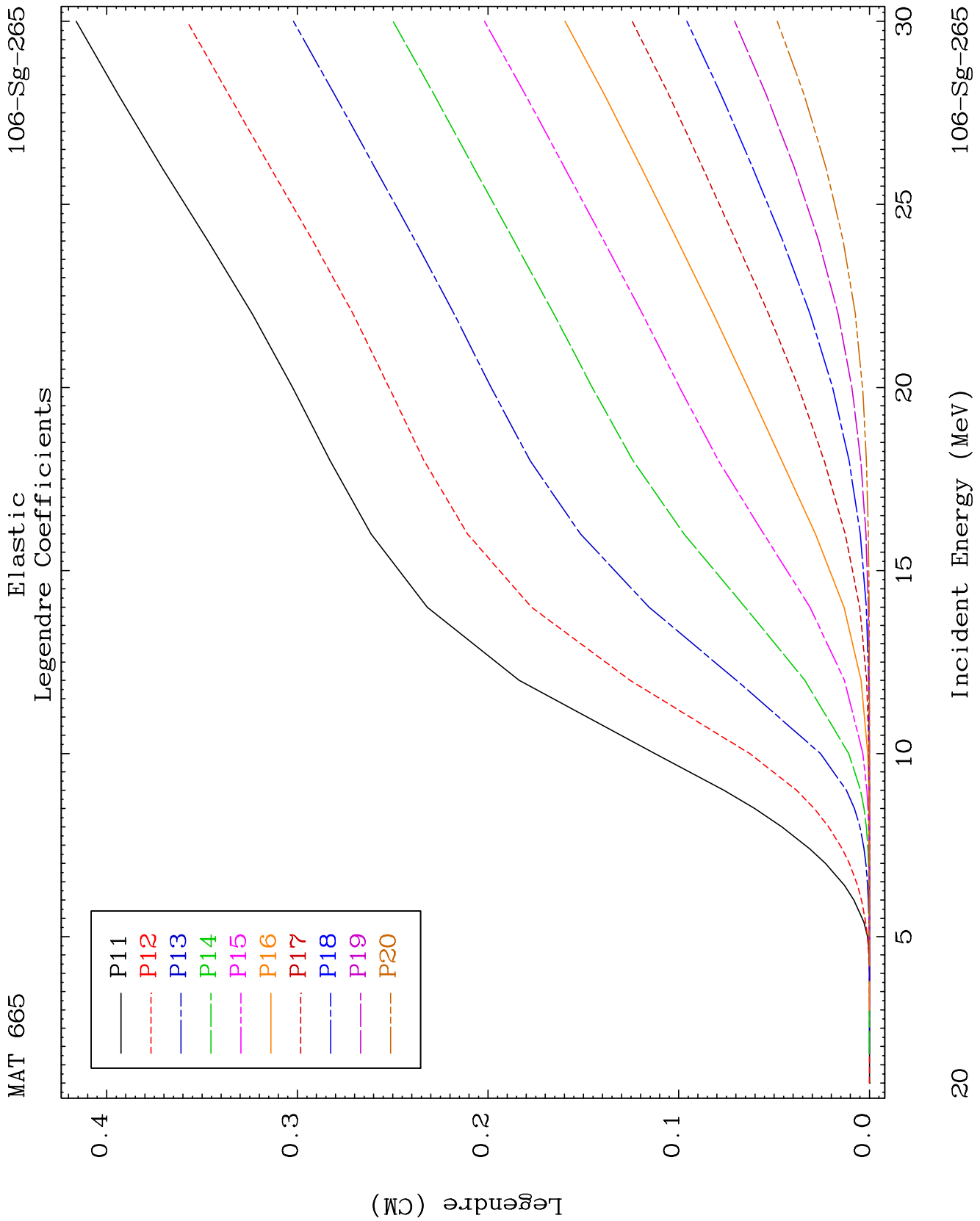








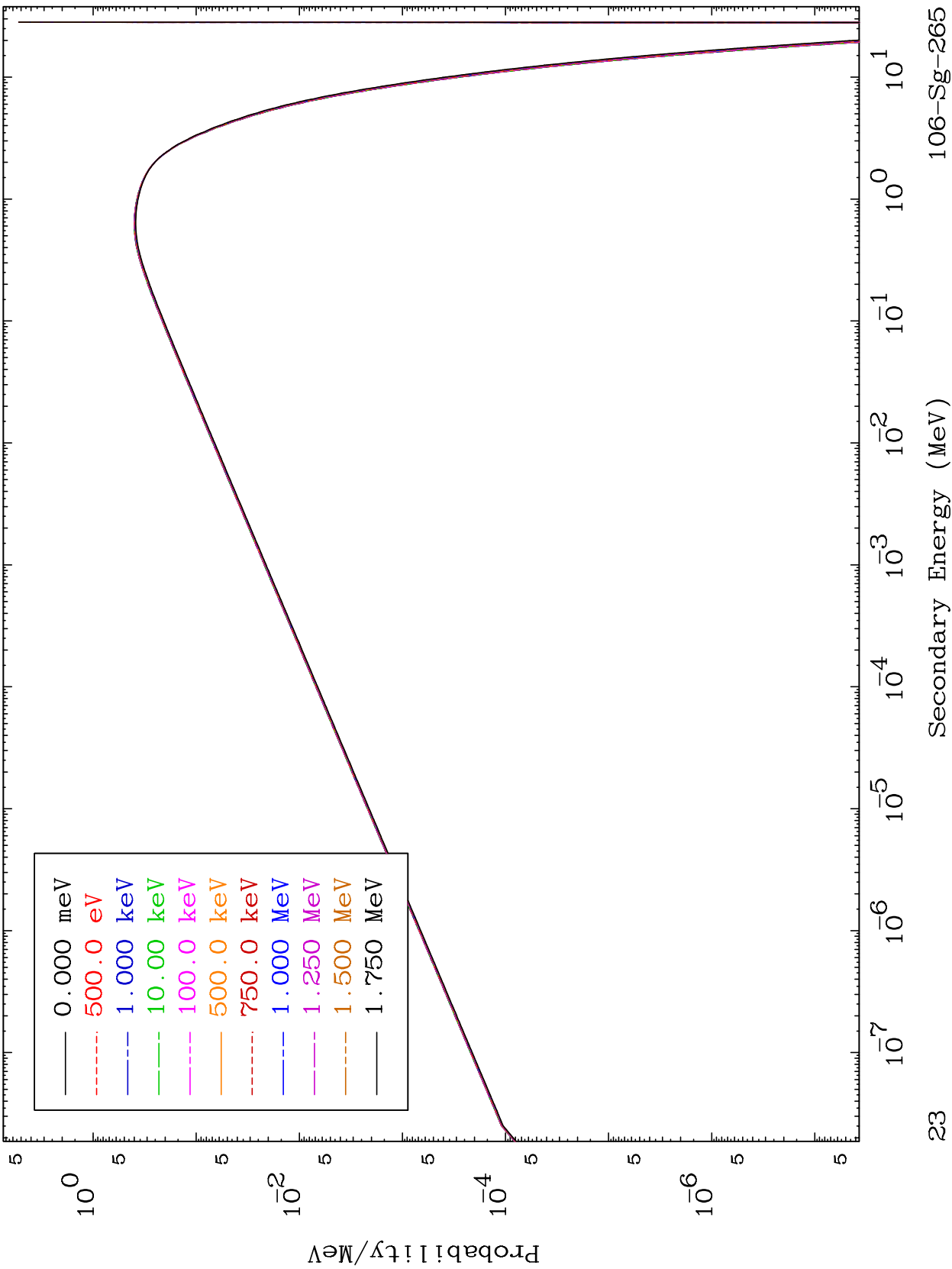






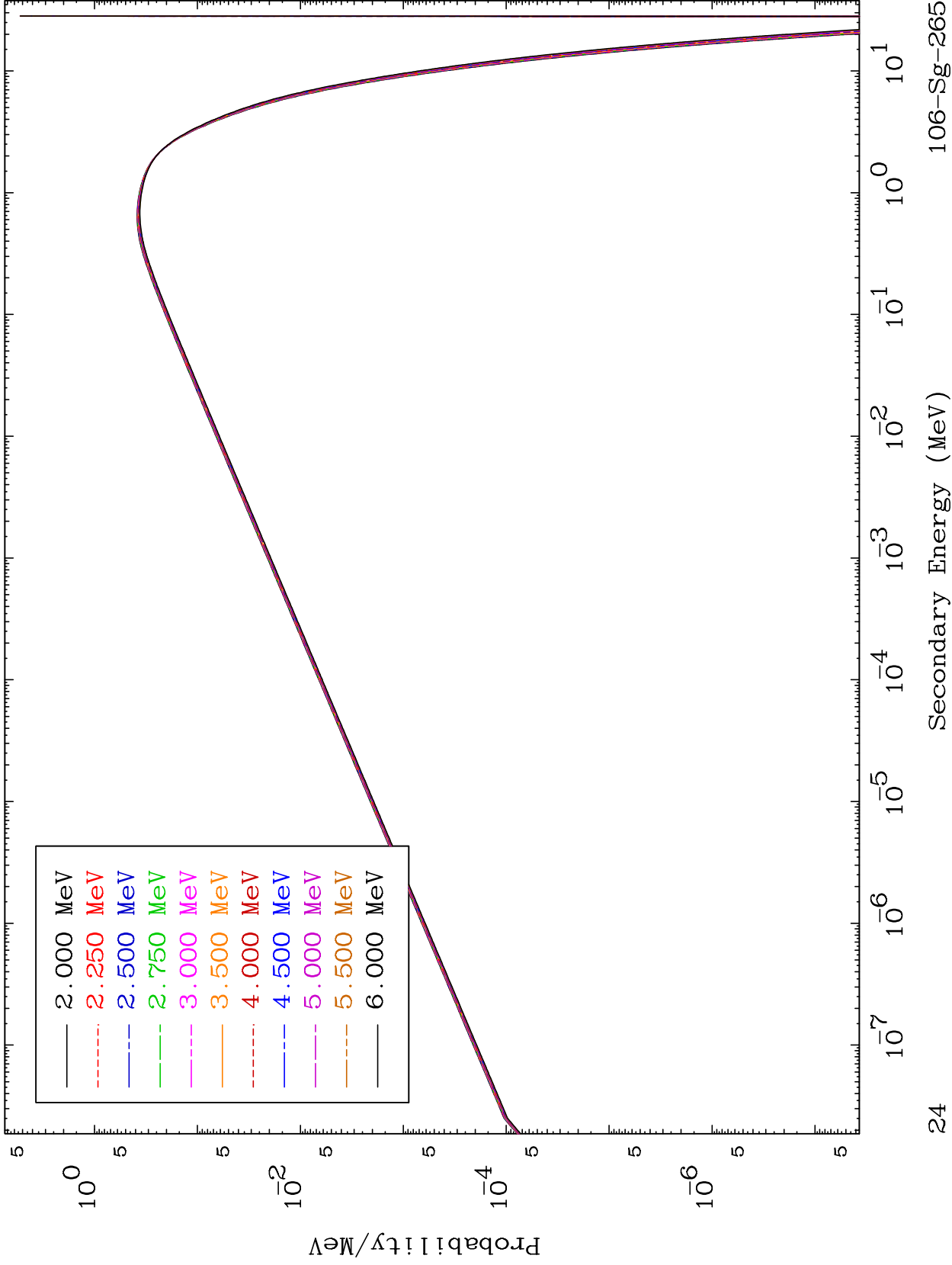


Fission Energy Distribution

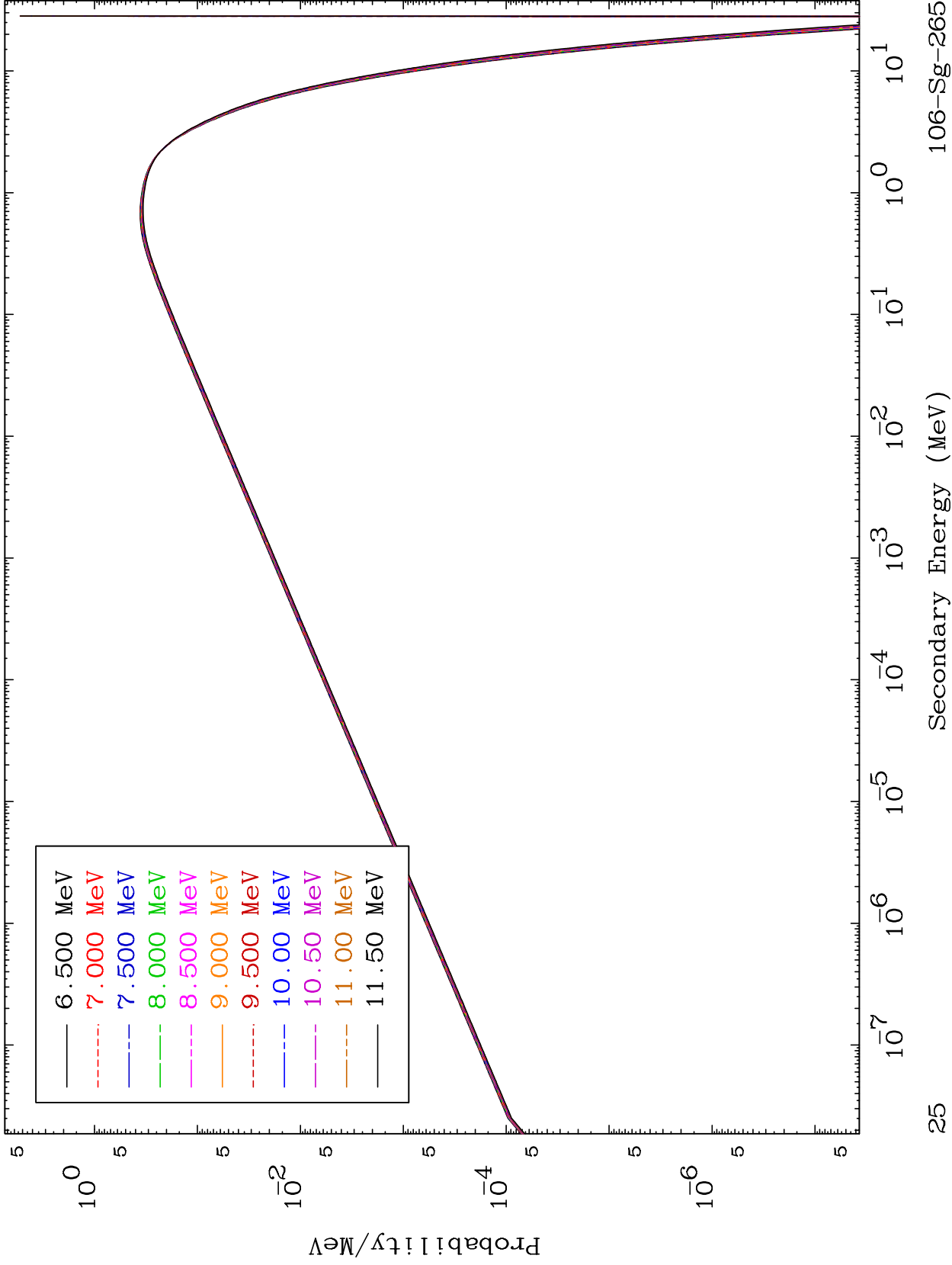




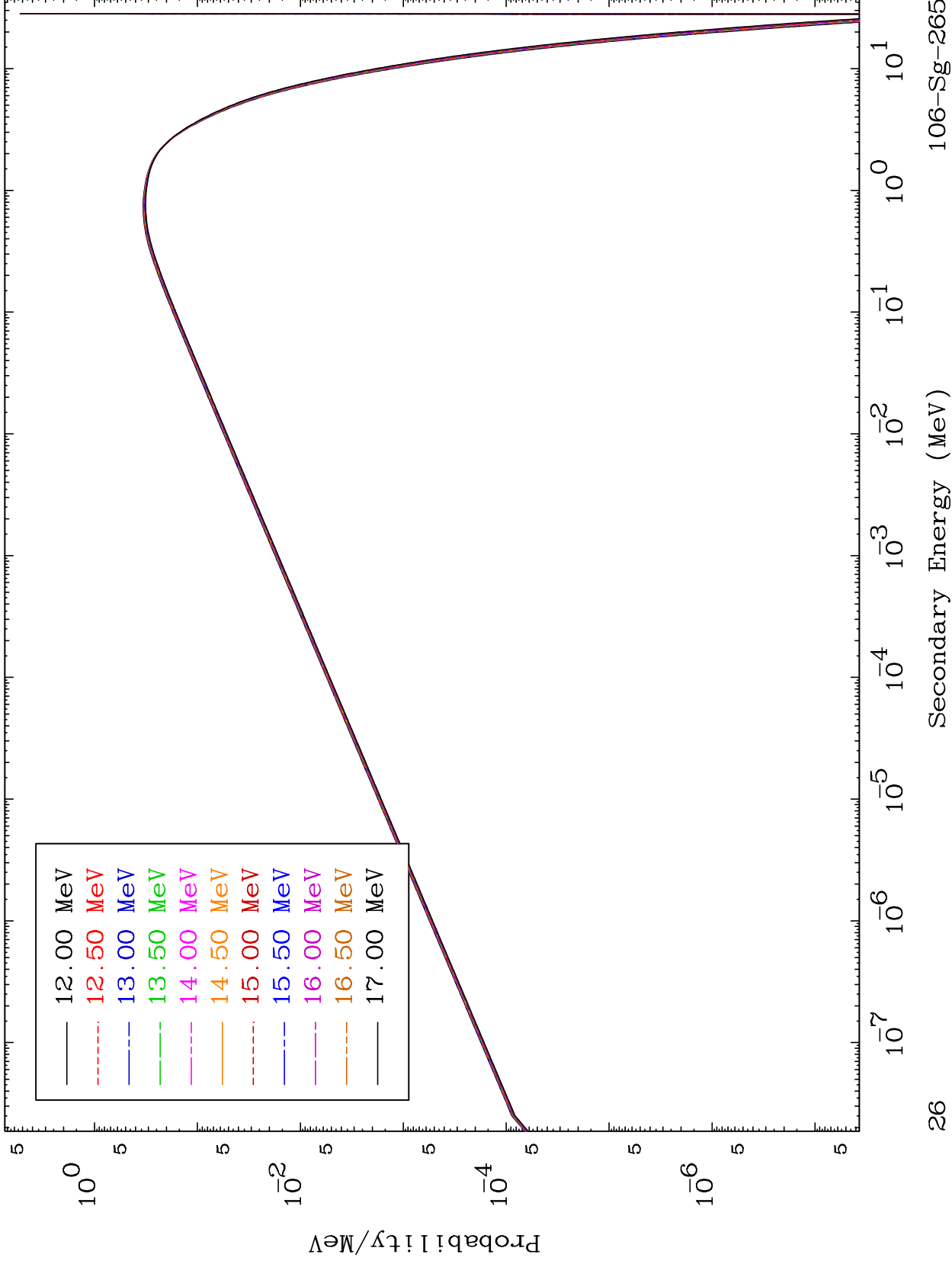
Fission  
Energy Distribution



Fission  
Energy Distribution



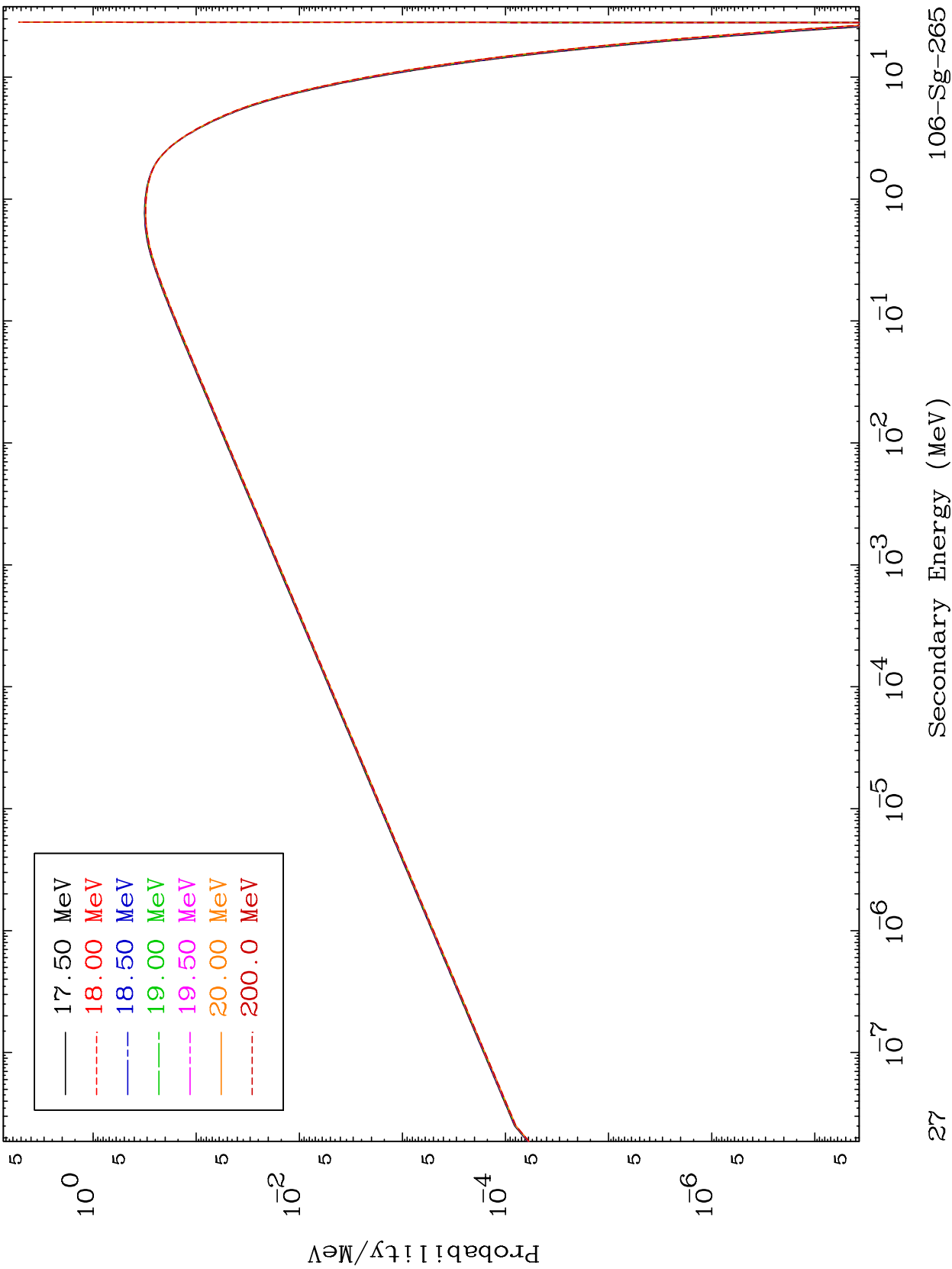
Fission  
Energy Distribution

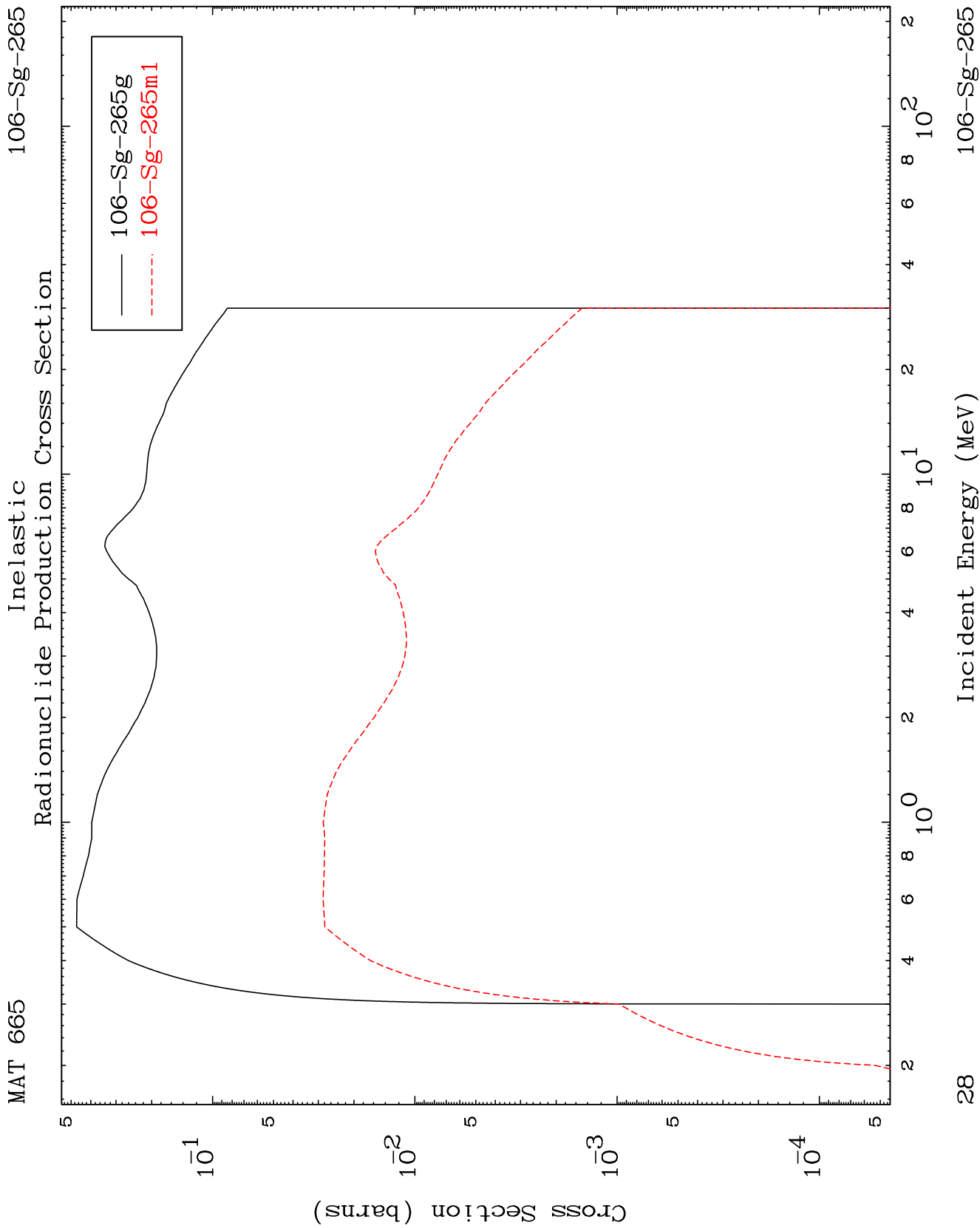


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### Fission Energy Distribution

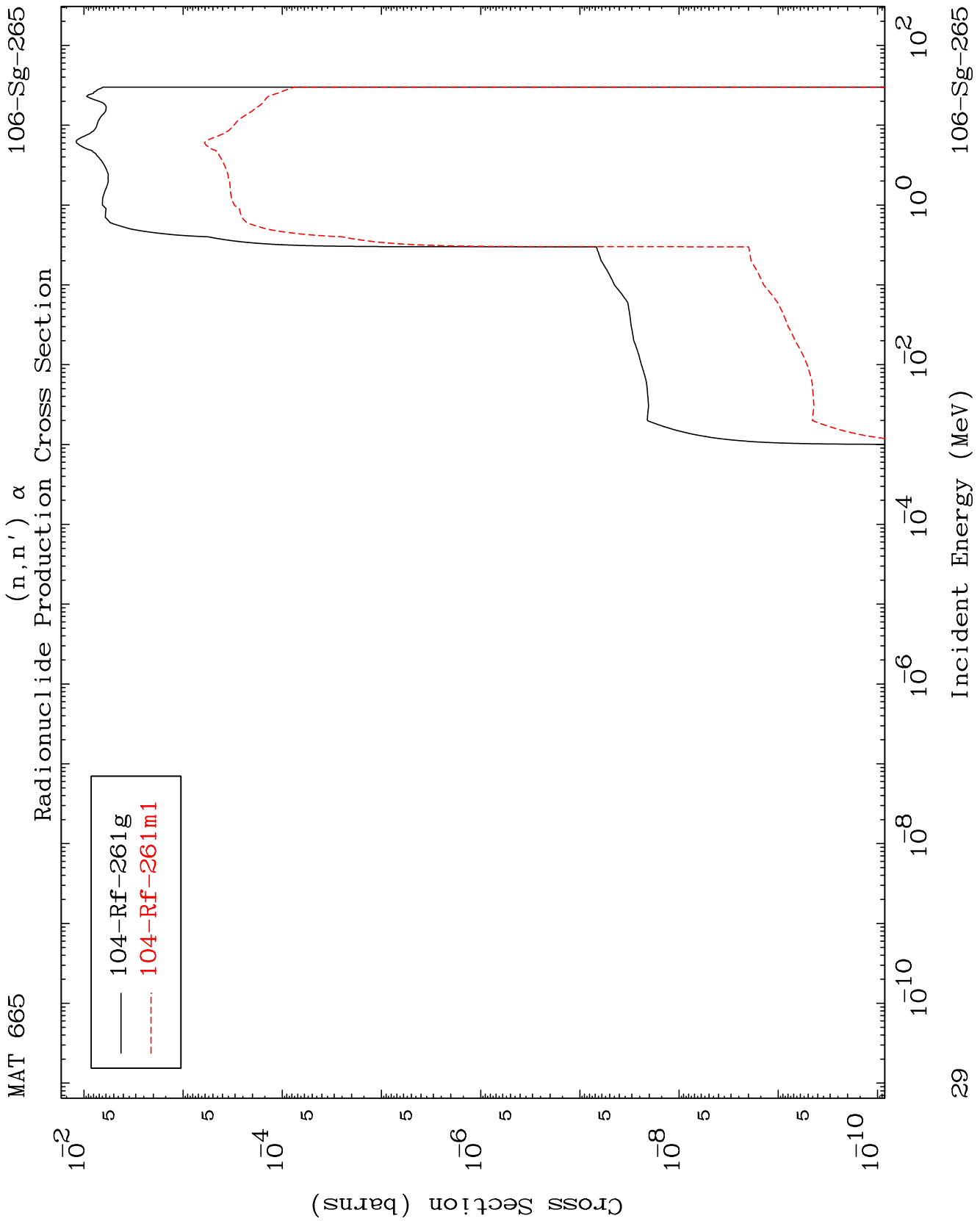
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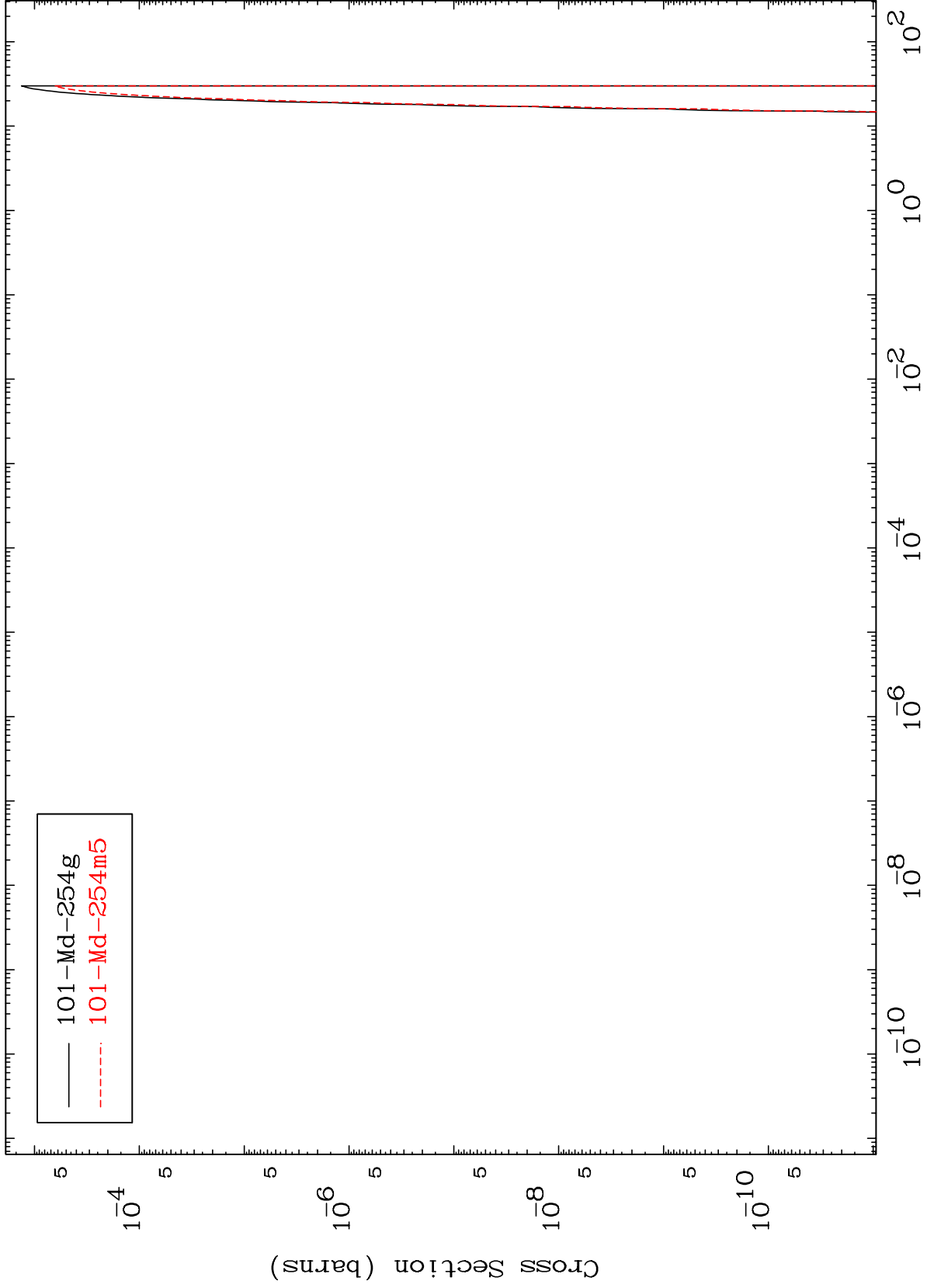


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(n,n') t,2 $\alpha$

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Radionuclide Production Cross Section



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Incident Energy (MeV)

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