

Program EVALPLOT  
(Version 2015-2)

by

Dermott E. Cullen  
(Present Contact Information)

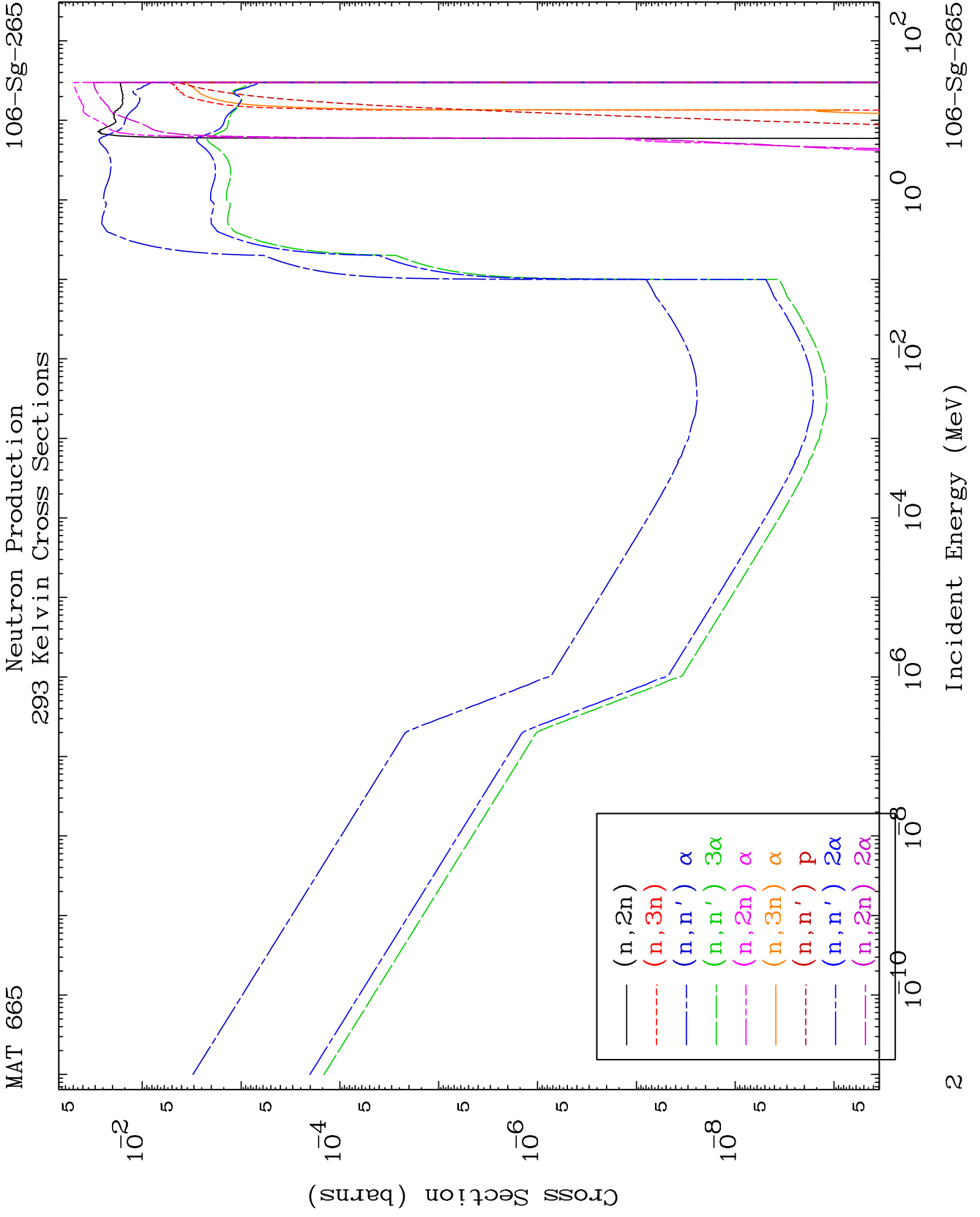
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Press Mouse Button to Start

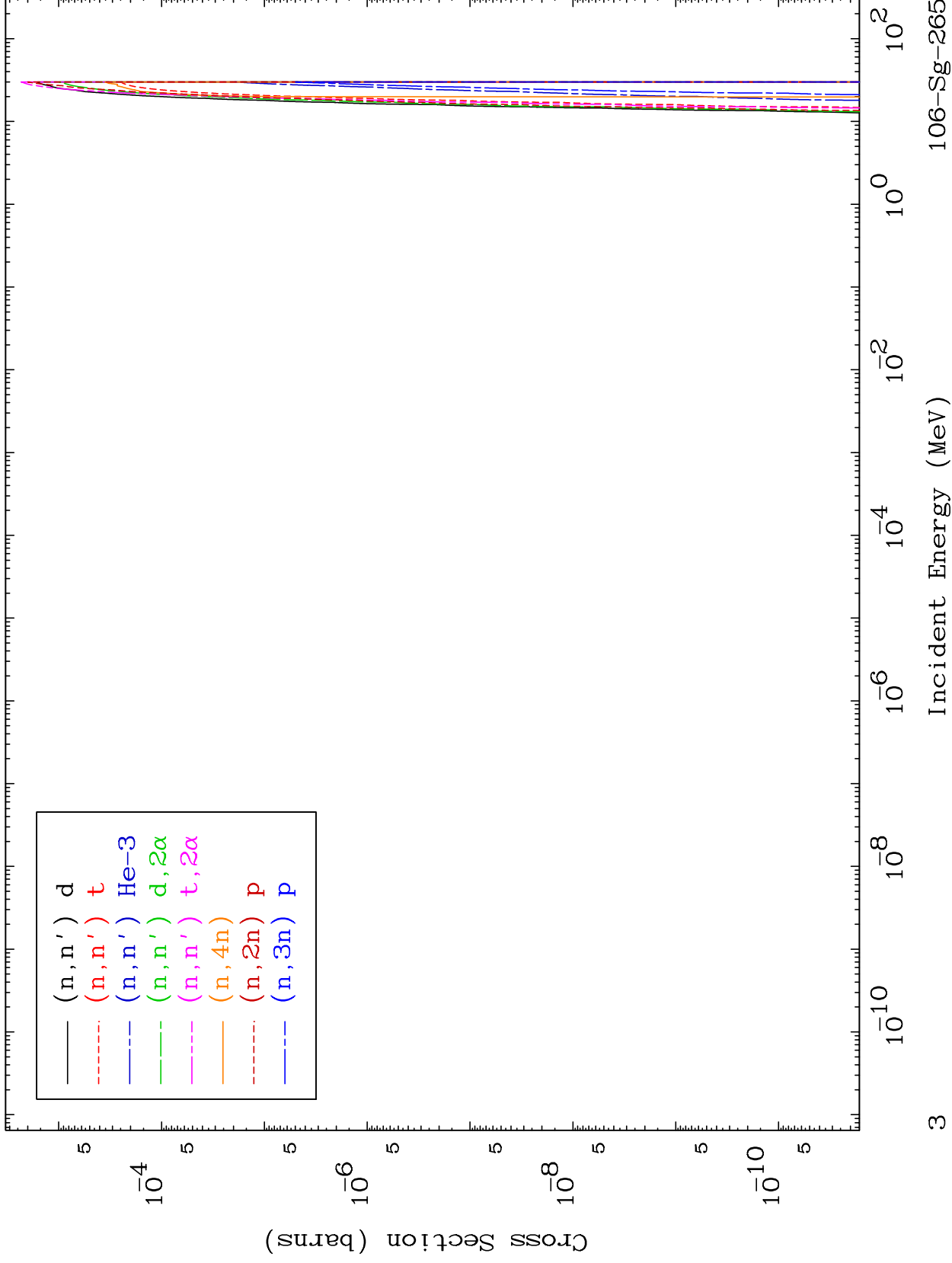


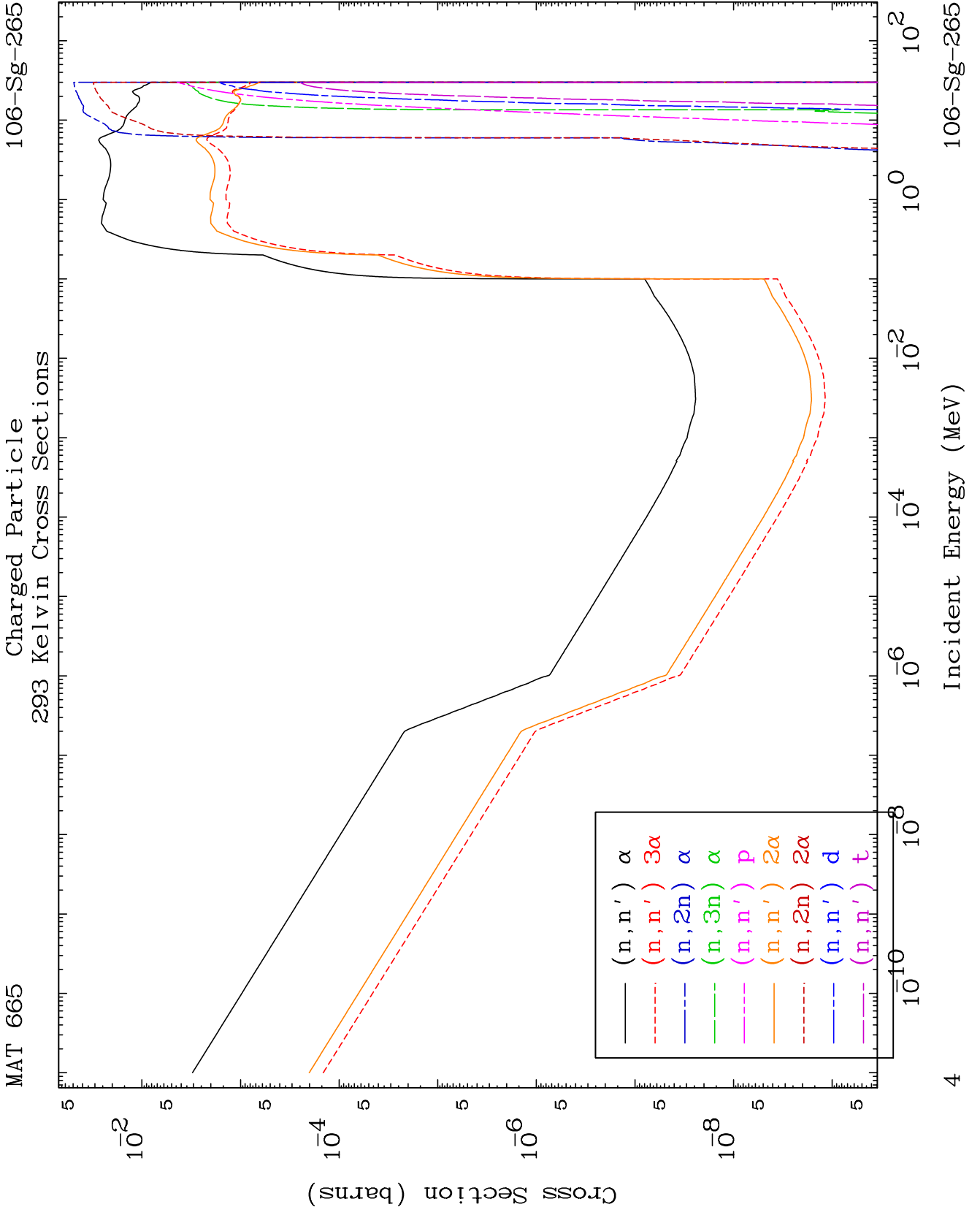


MAT 665

Neutron Production  
293 Kelvin Cross Sections

106-Sg-265

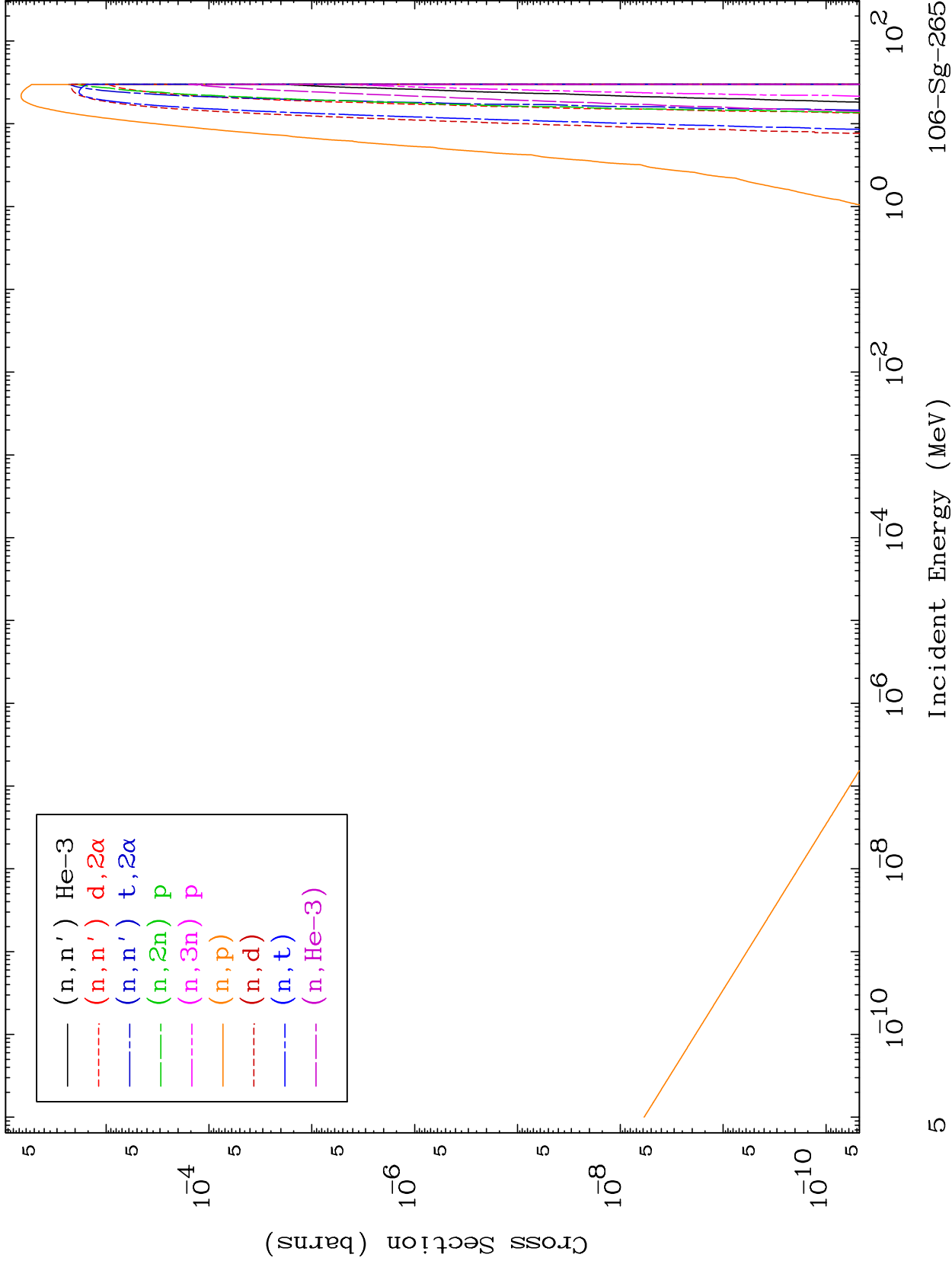




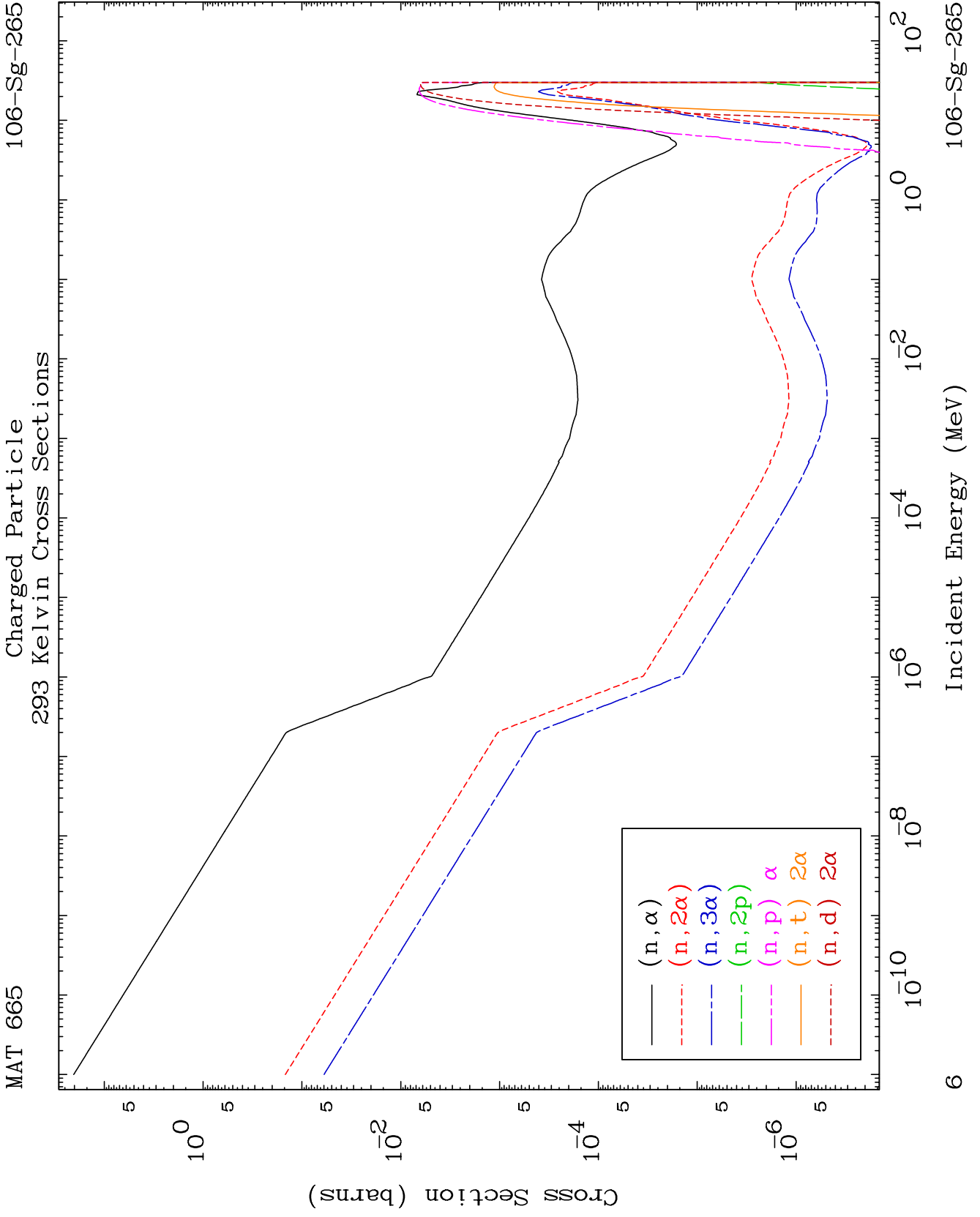
MAT 665

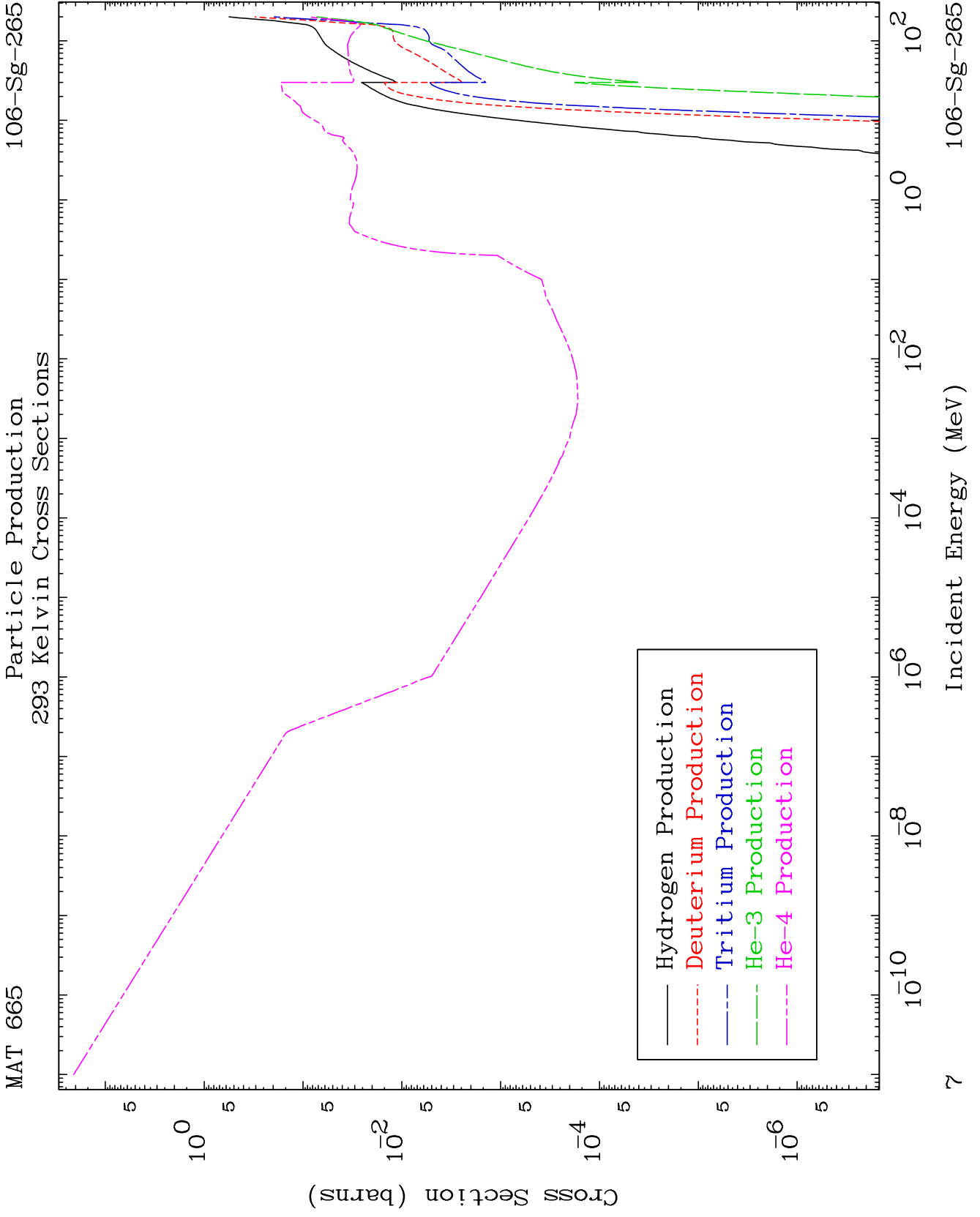
Charged Particle  
293 Kelvin Cross Sections

106-Sg-265



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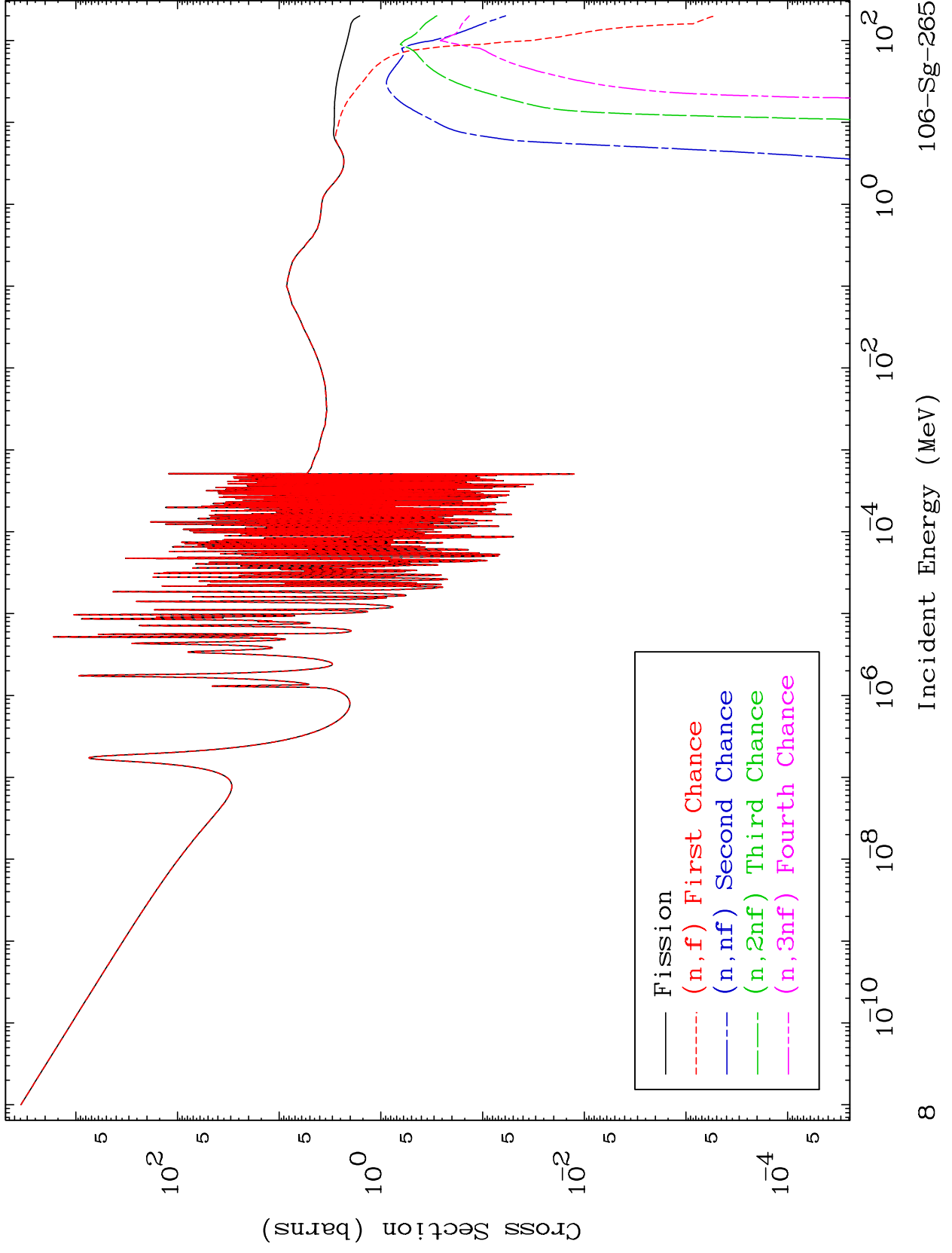


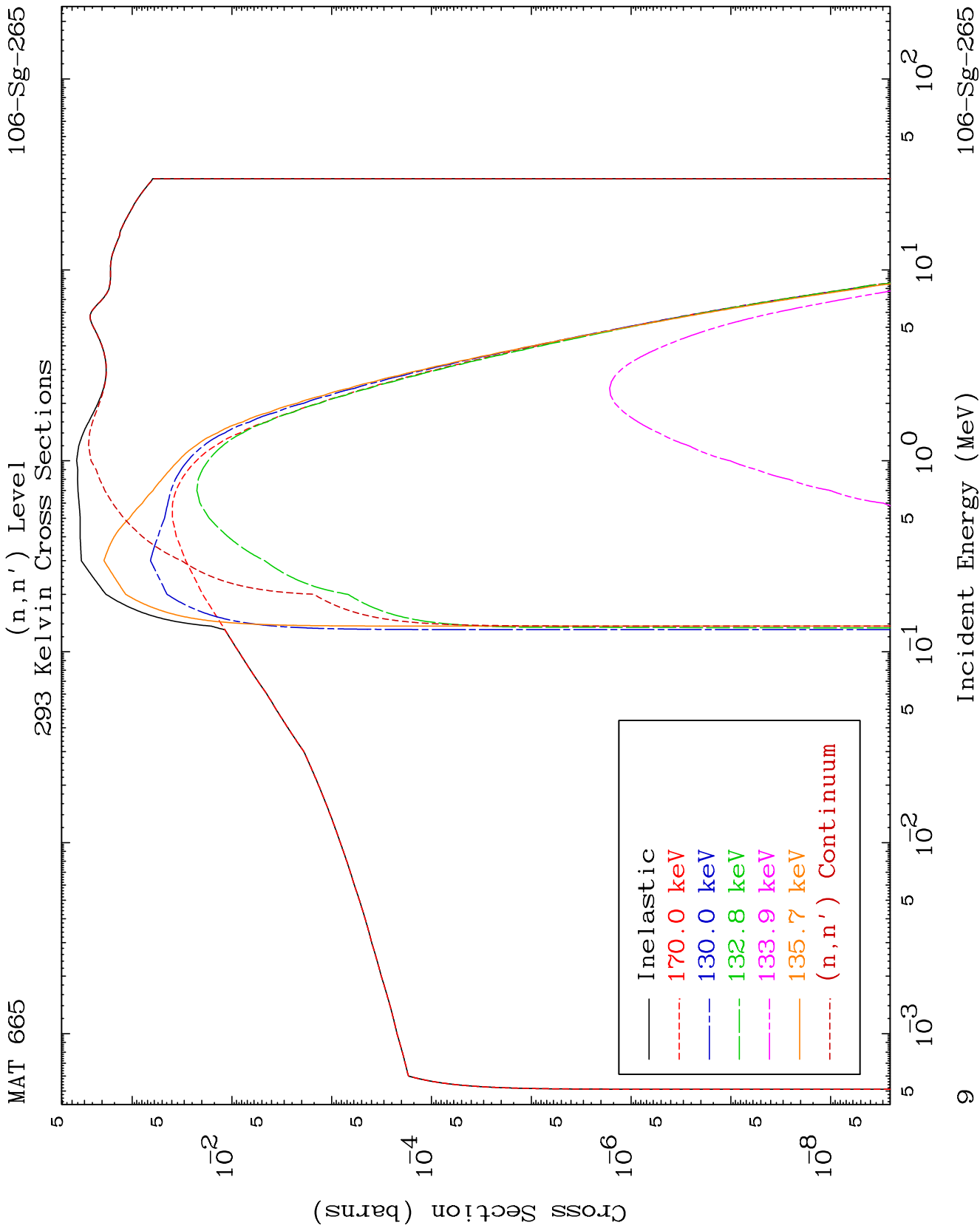


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Fission  
293 Kelvin Cross Sections

106-Sg-265

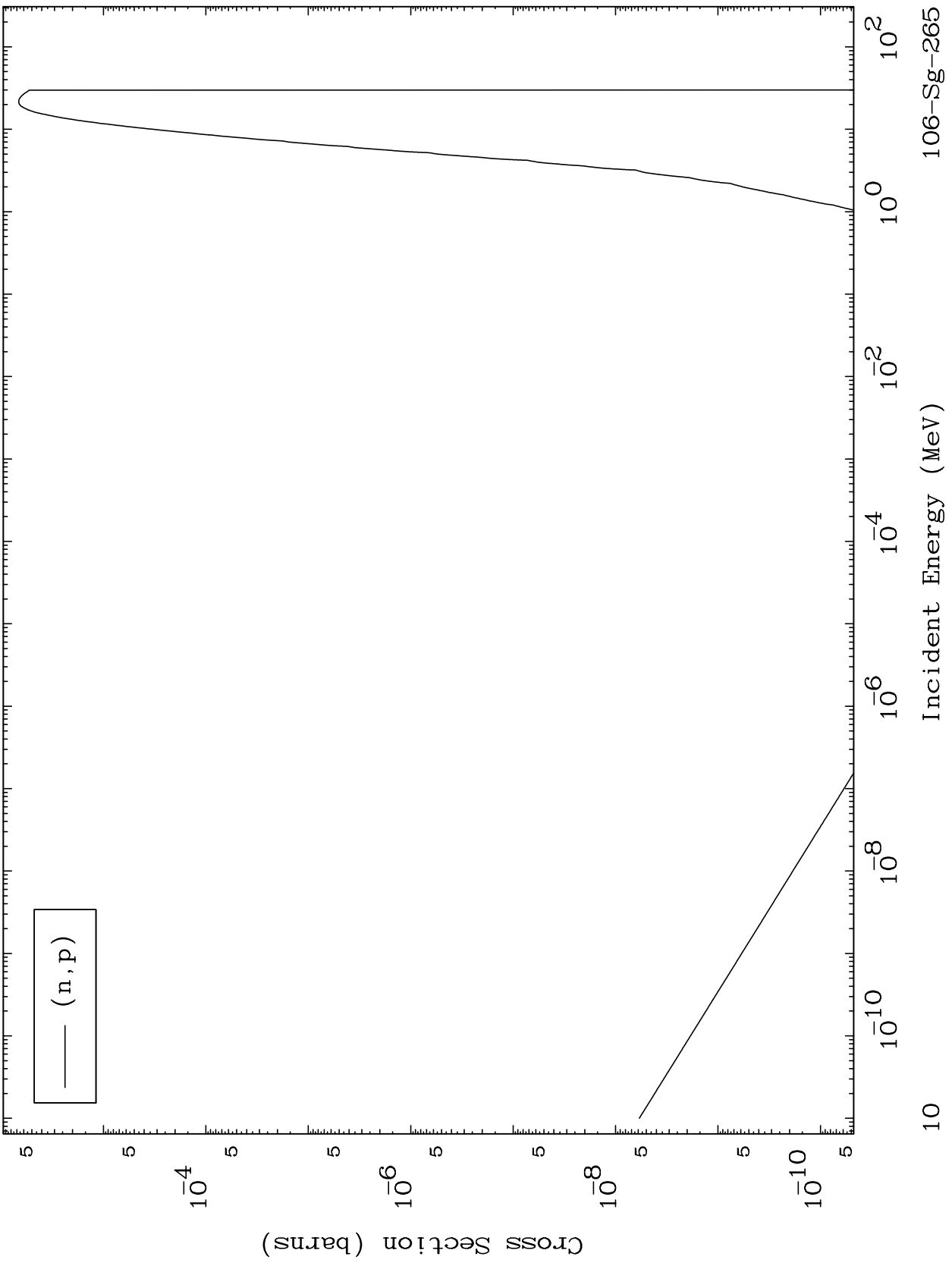




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(n,p) Levels  
293 Kelvin Cross Sections

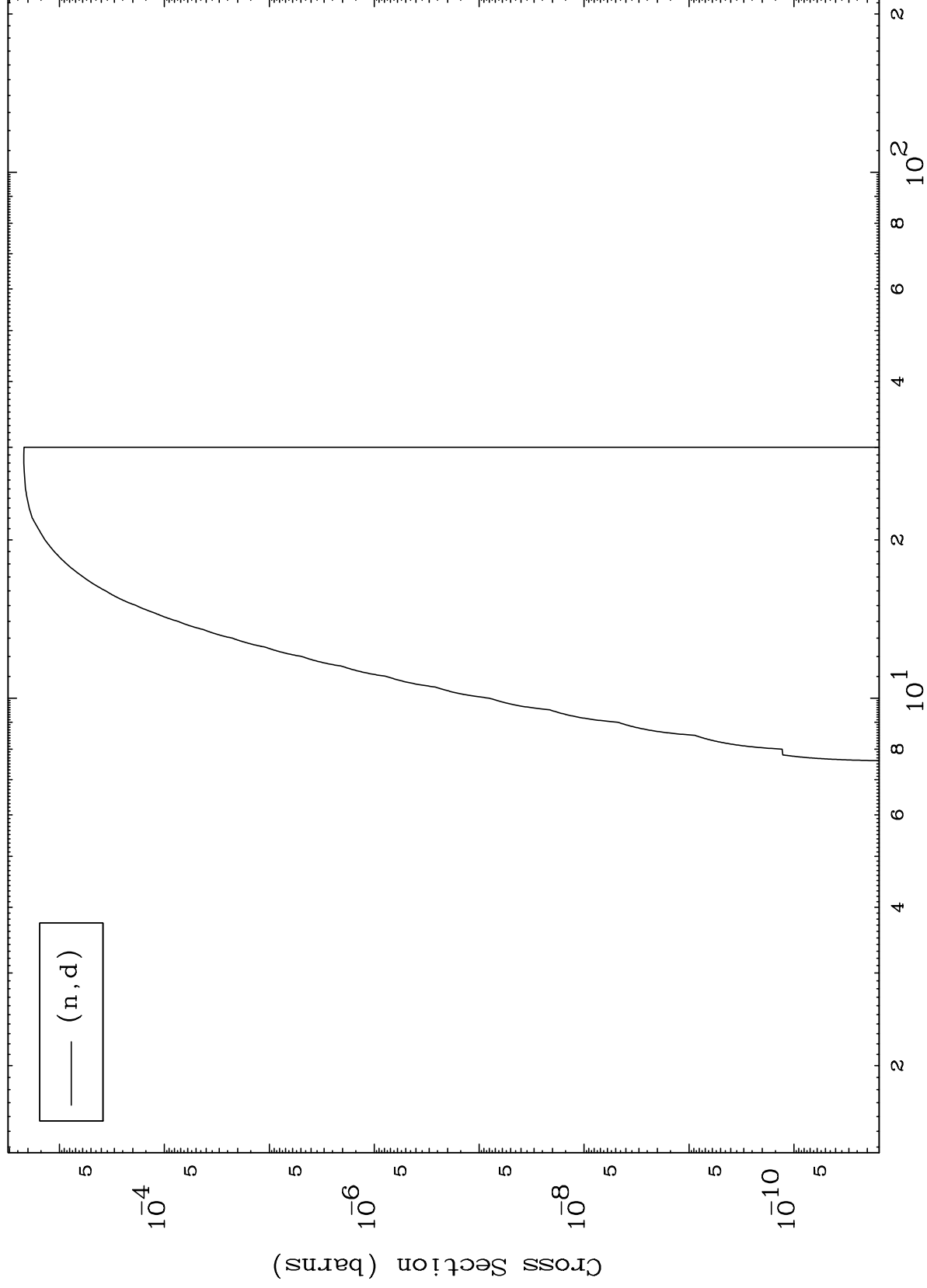
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(n,d) Levels  
293 Kelvin Cross Sections

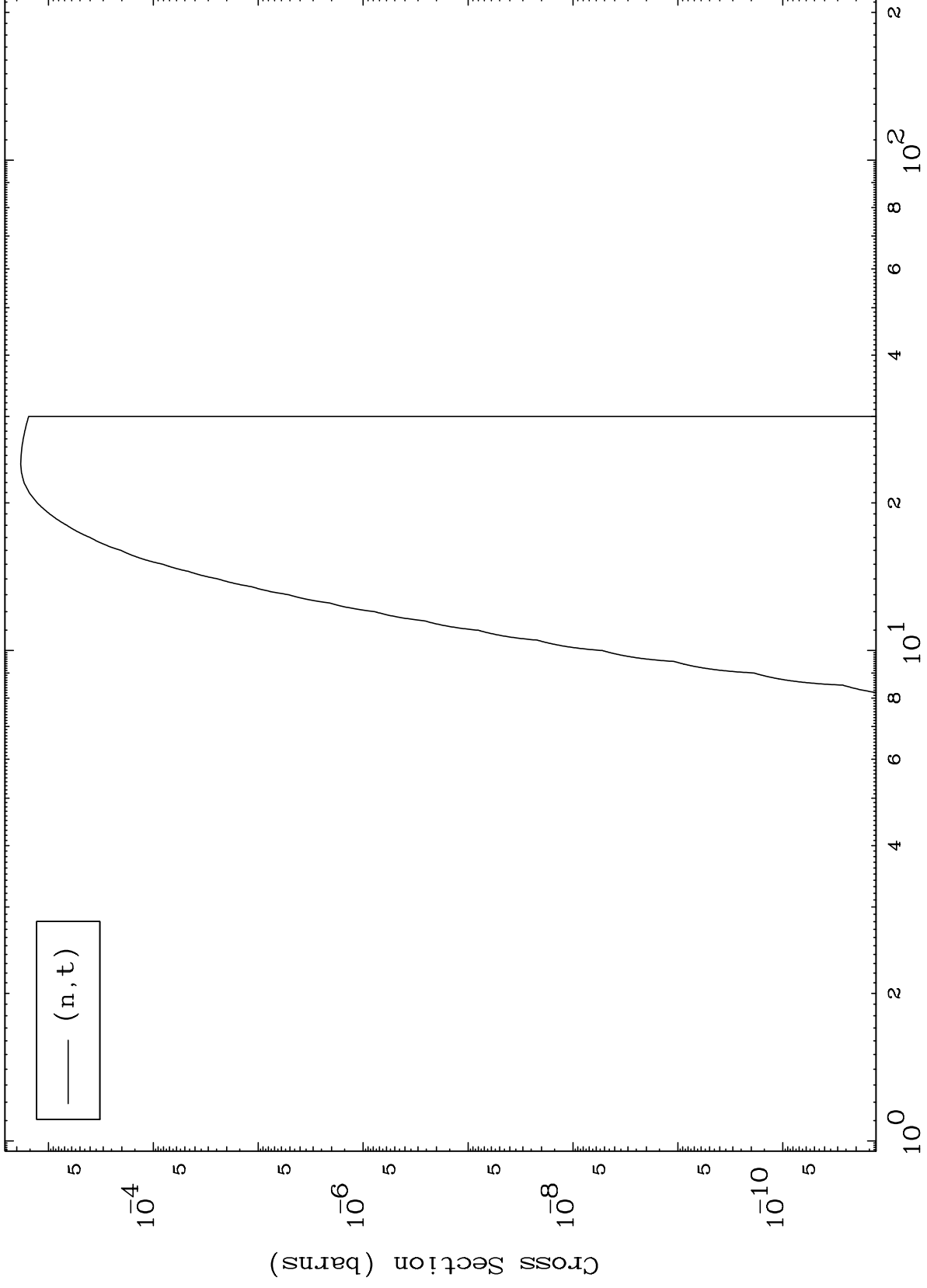
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(n,t) Levels  
293 Kelvin Cross Sections

106-Sg-265



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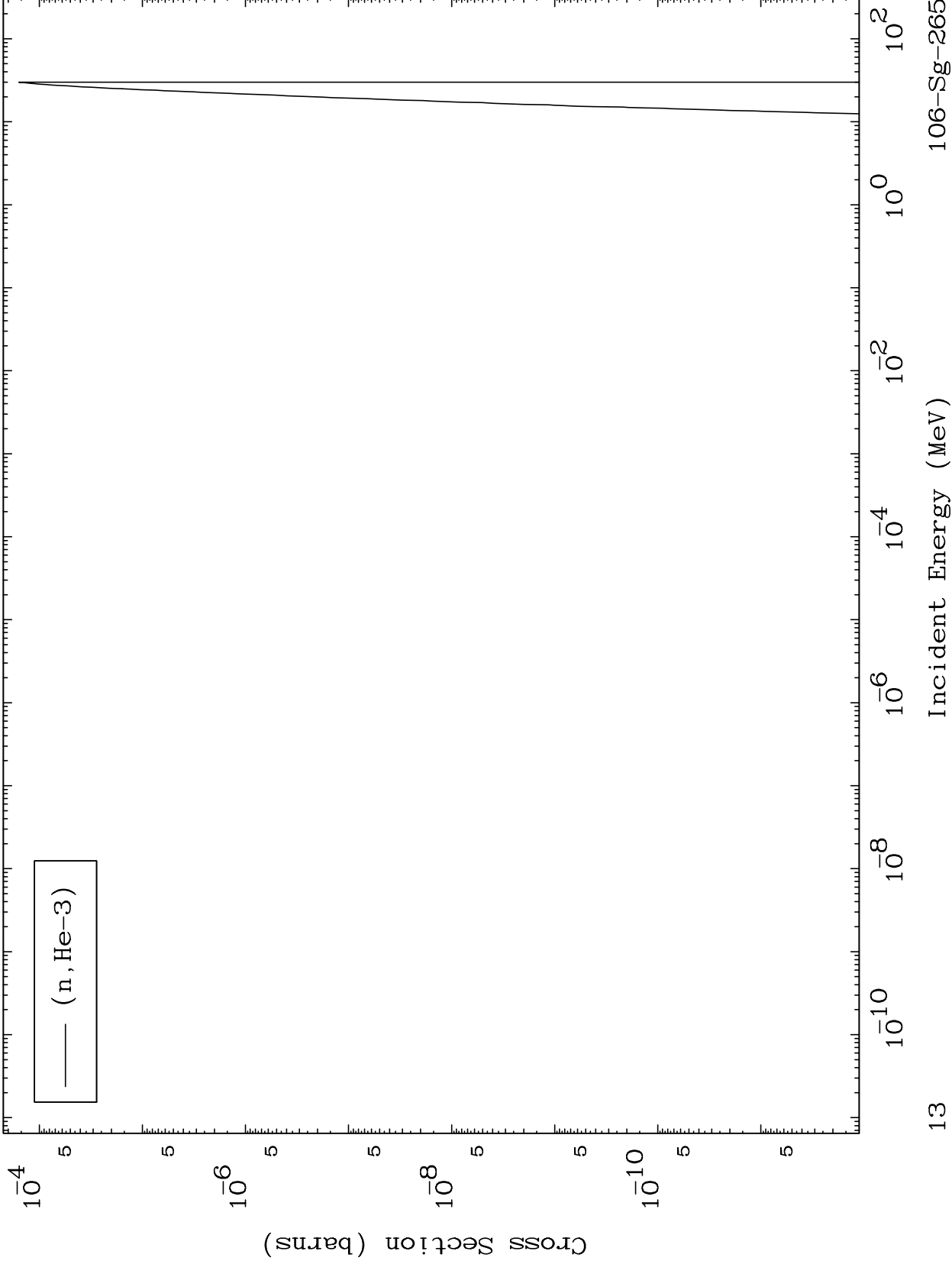
Incident Energy (MeV)

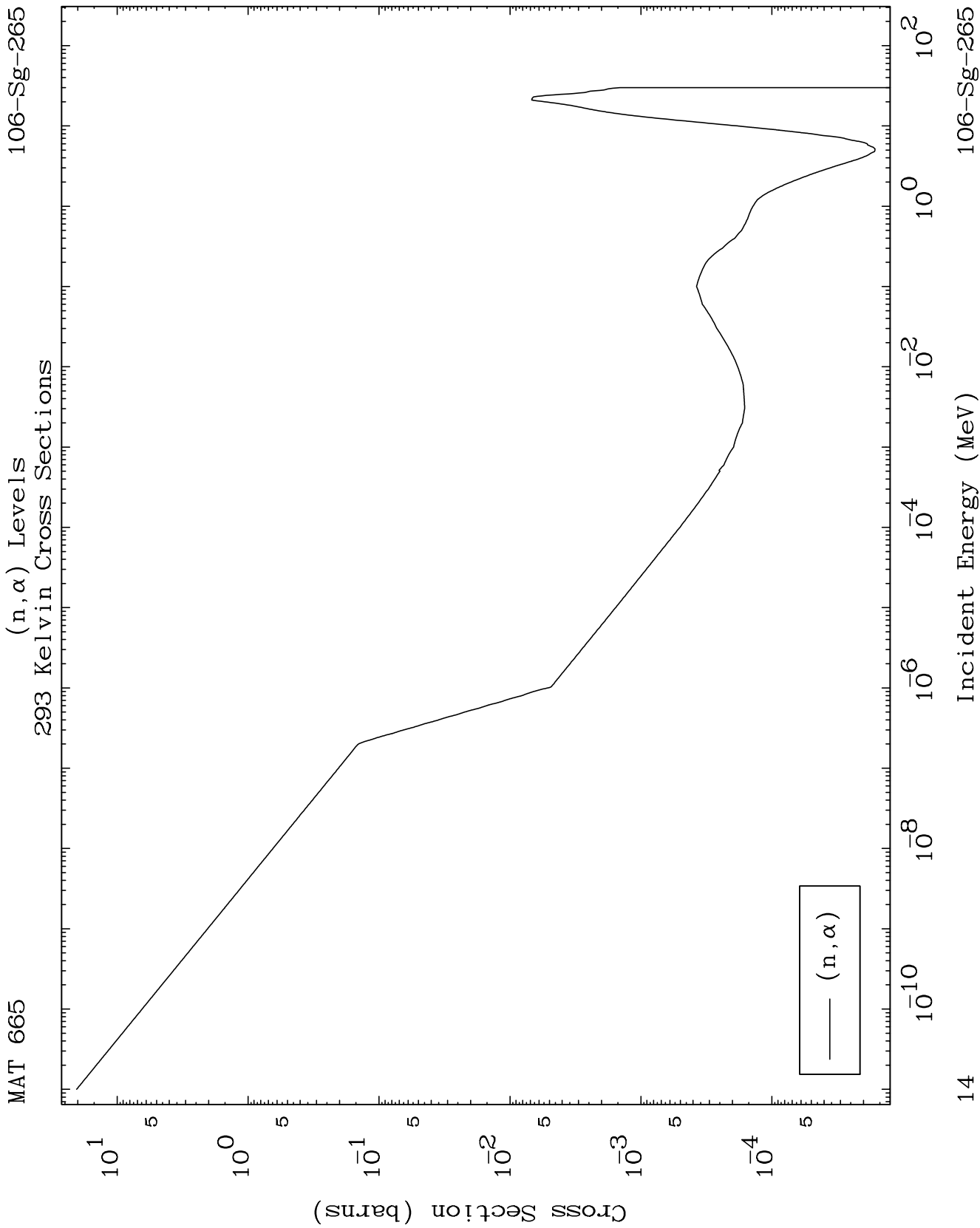
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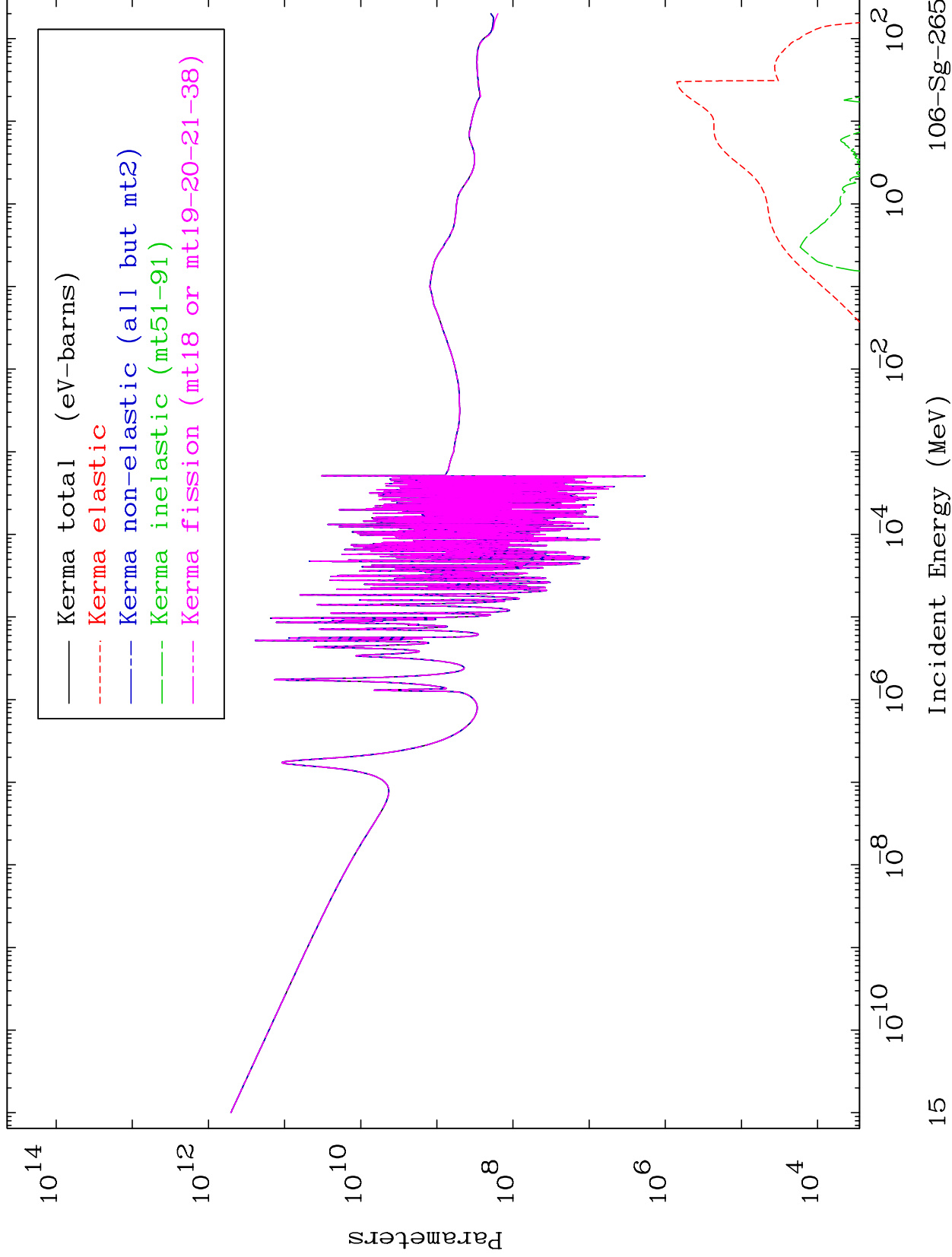
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(n,He3) Levels  
293 Kelvin Cross Sections

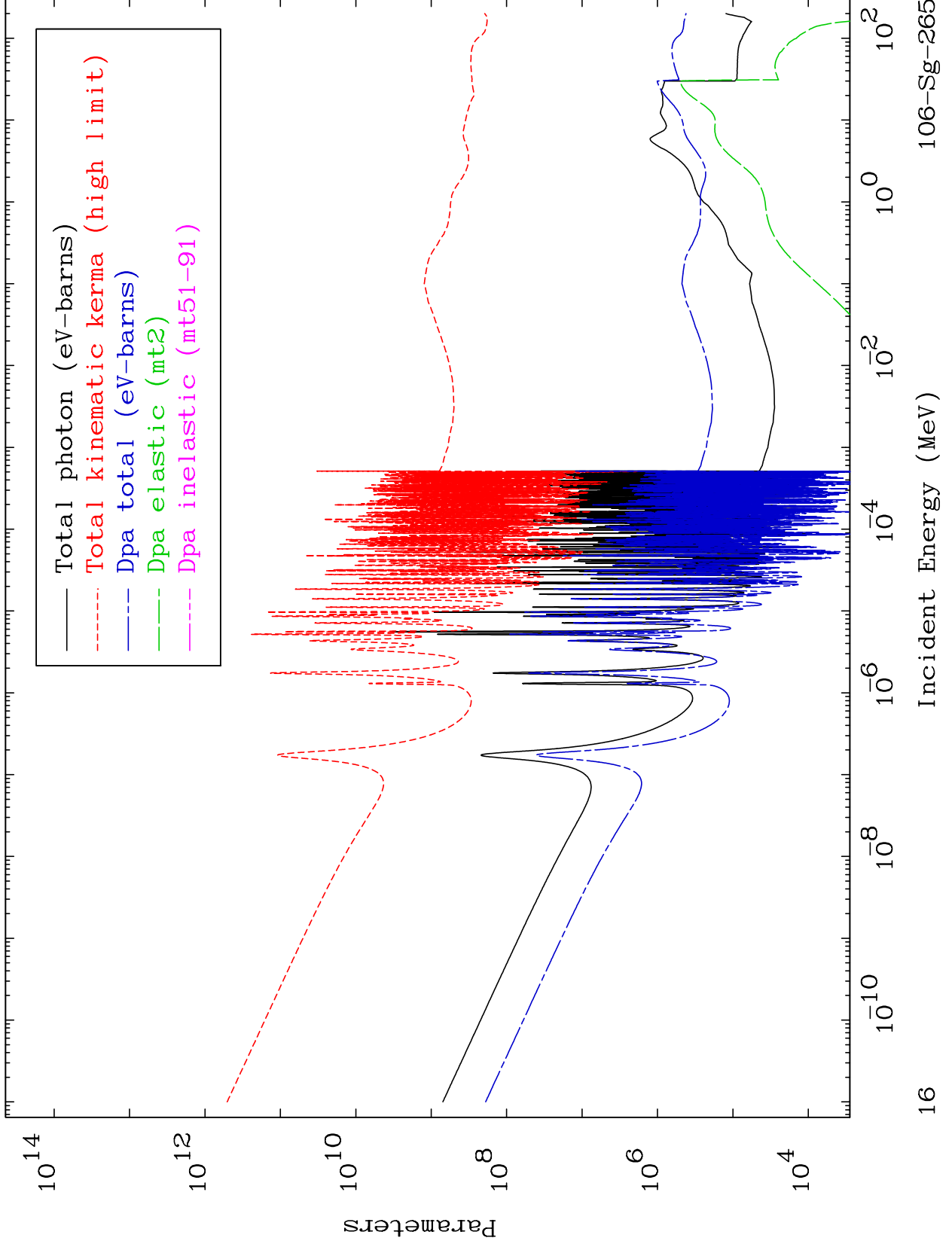
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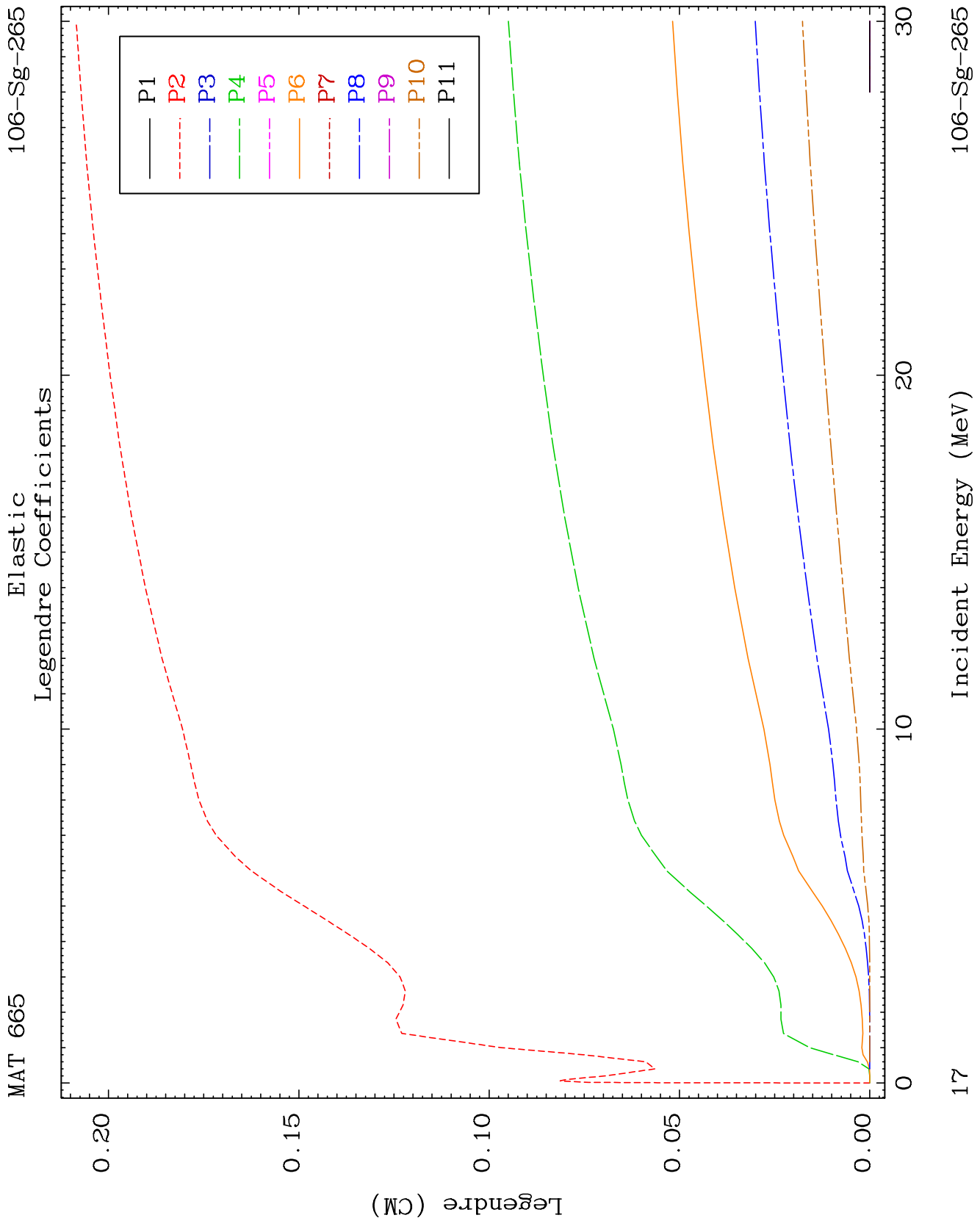


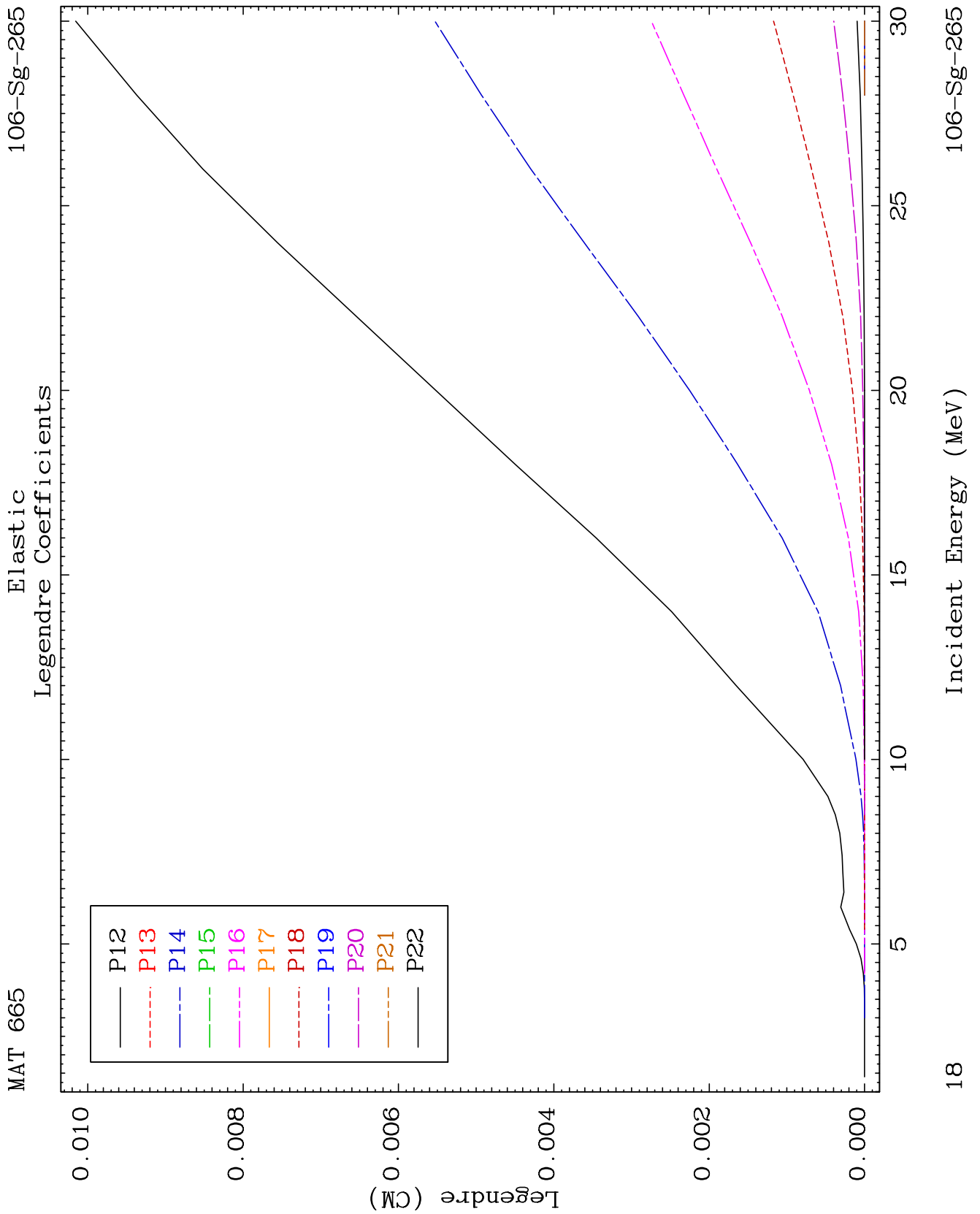






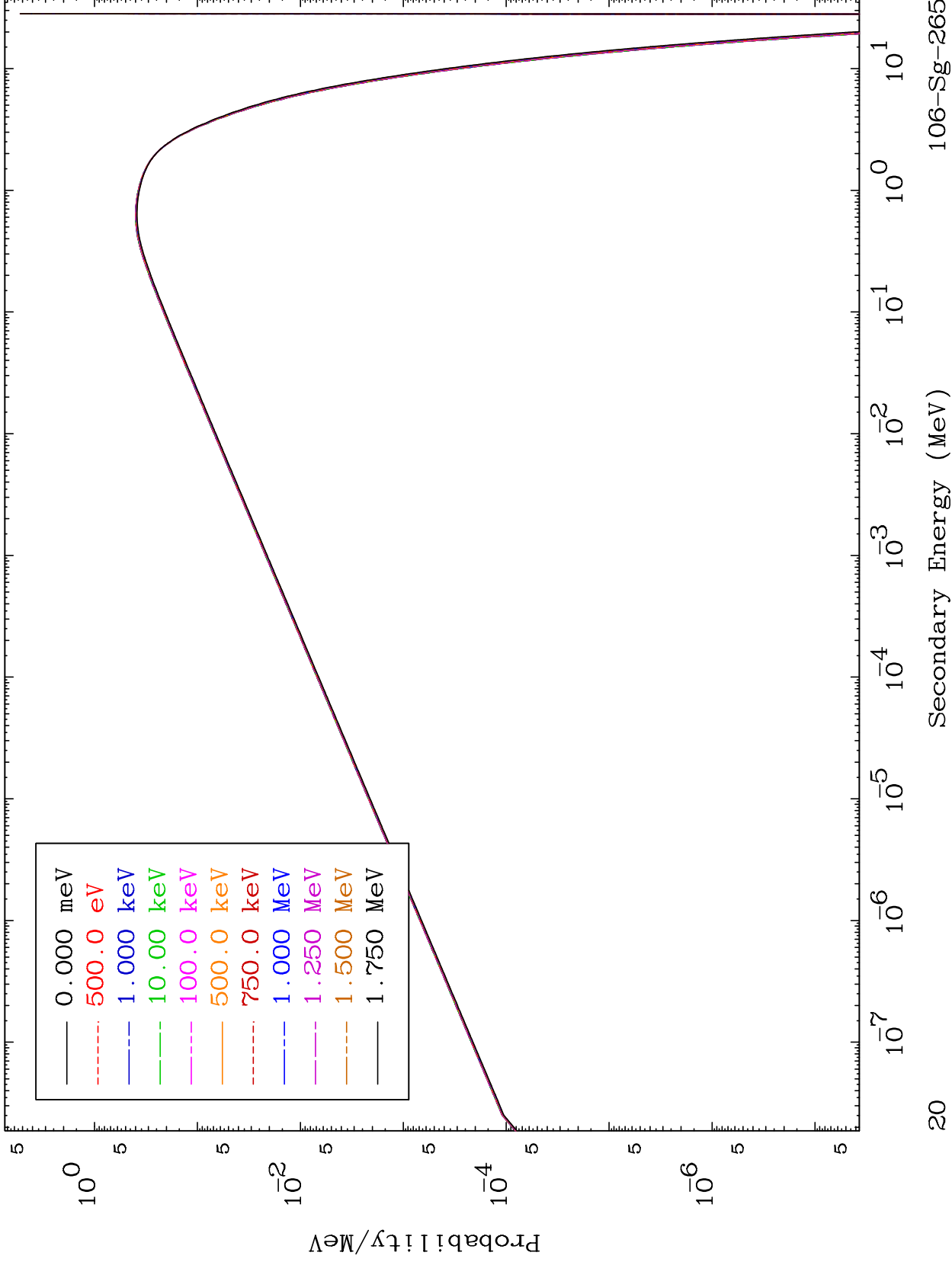




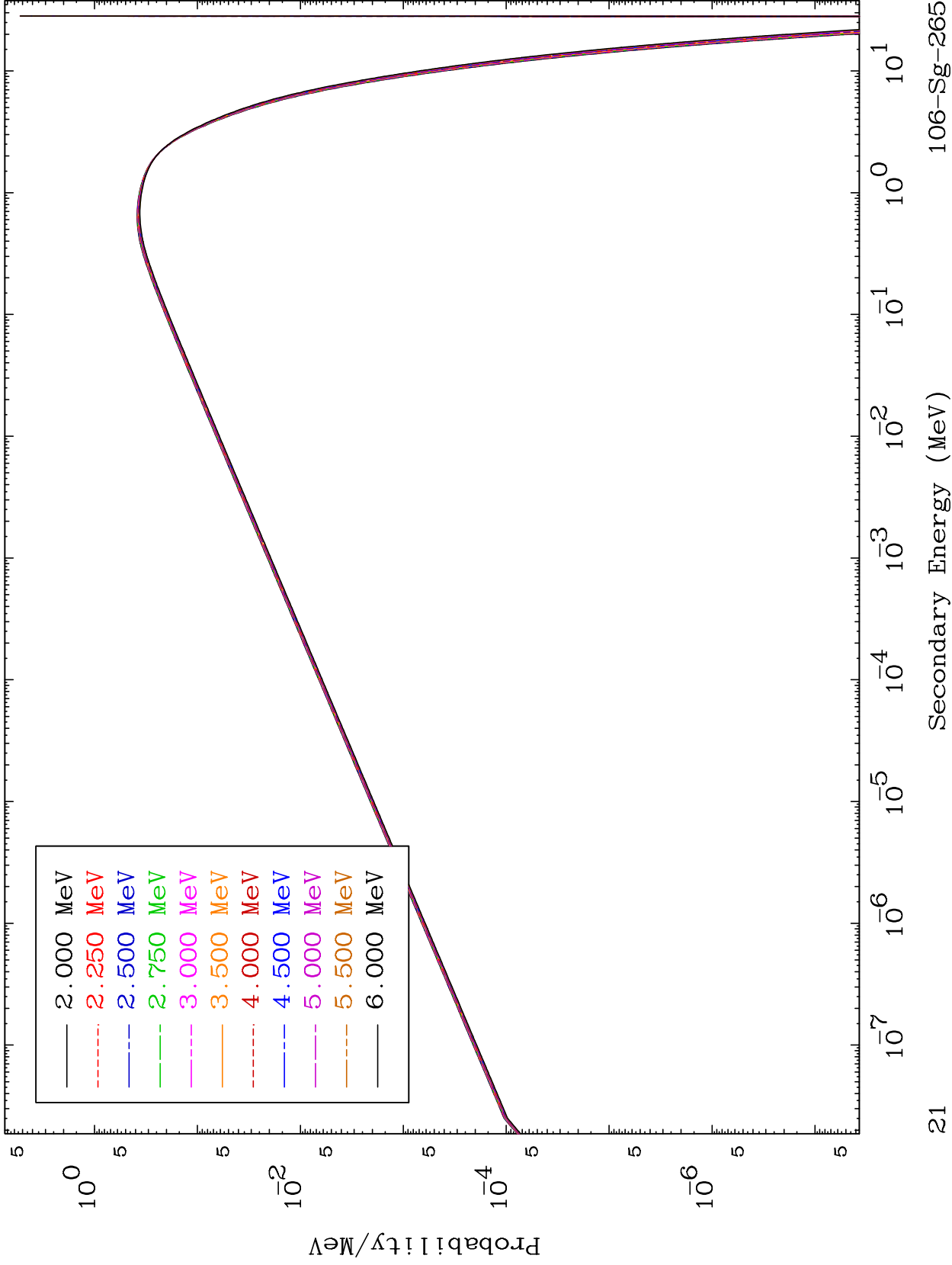




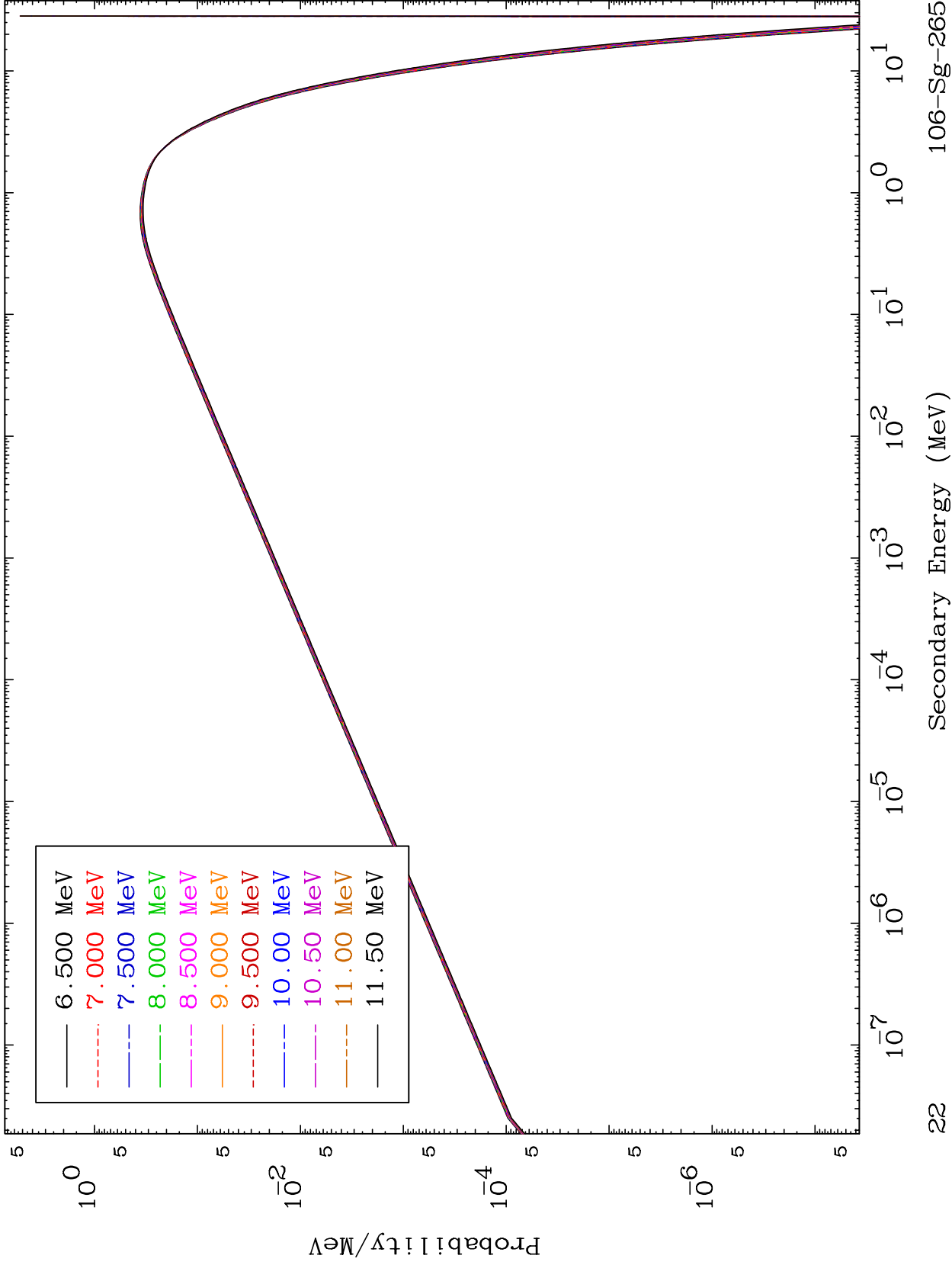
Fission Energy Distribution



Fission  
Energy Distribution



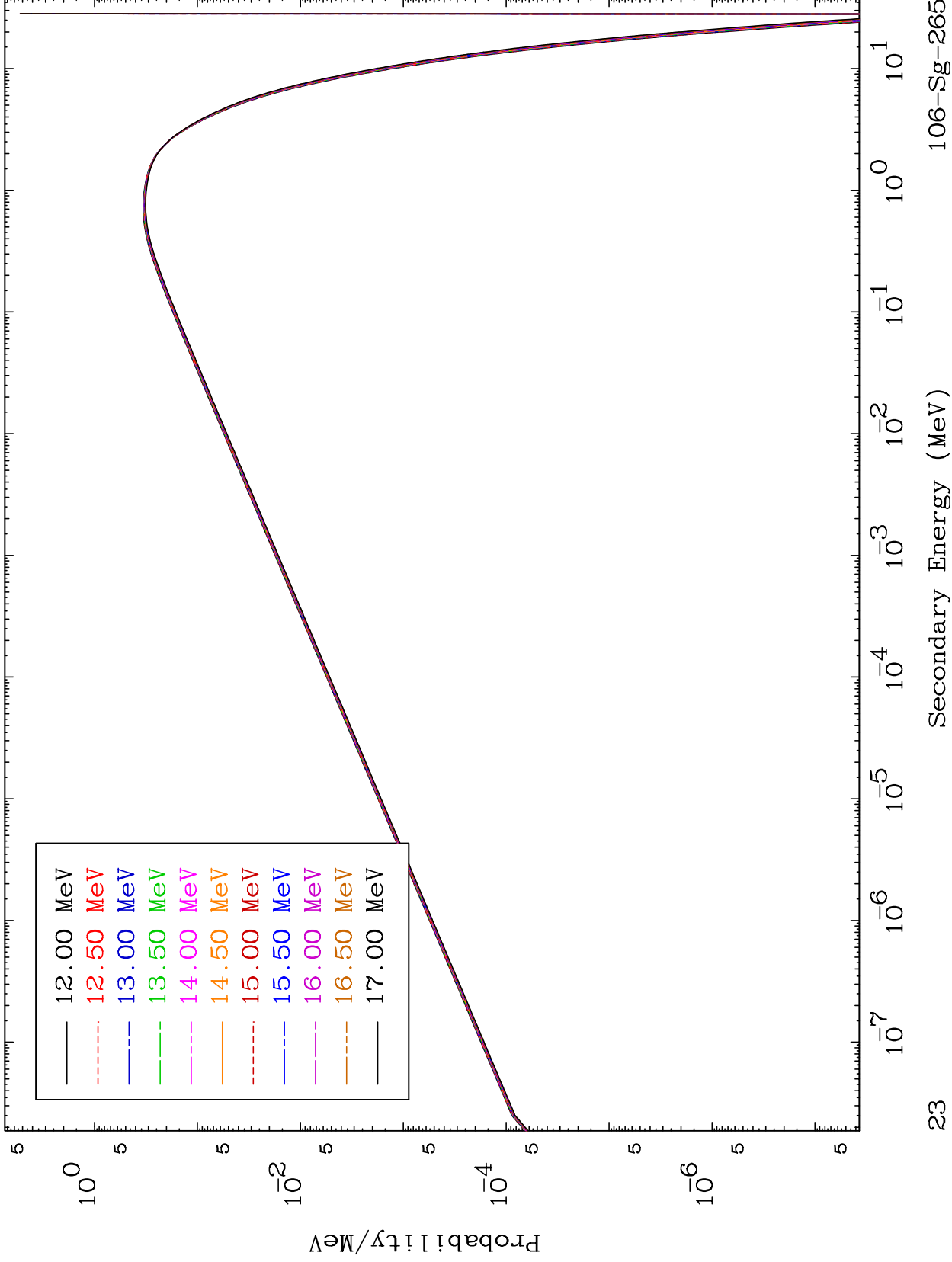
Fission  
Energy Distribution



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### Fission Energy Distribution

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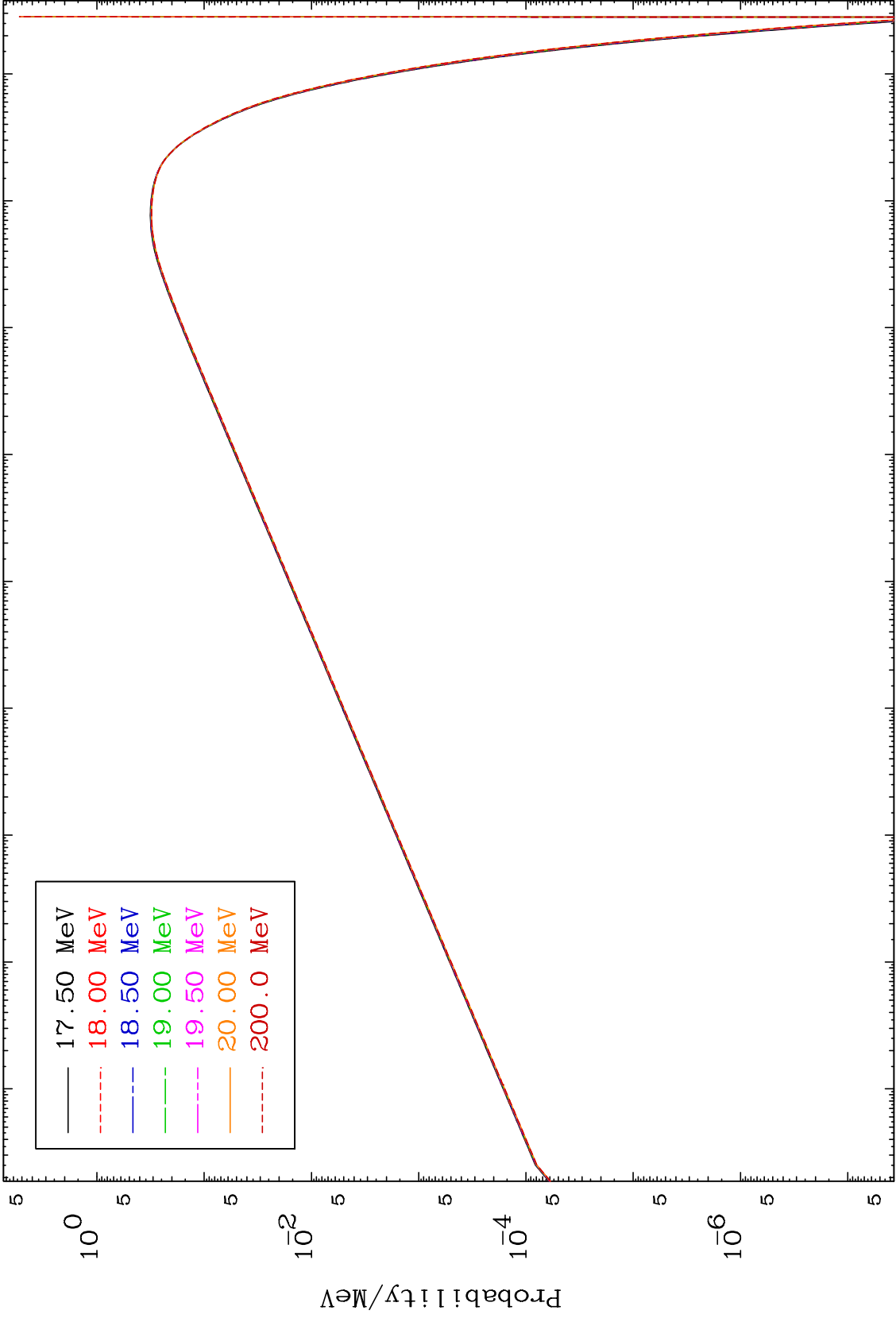




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# Fission Energy Distribution

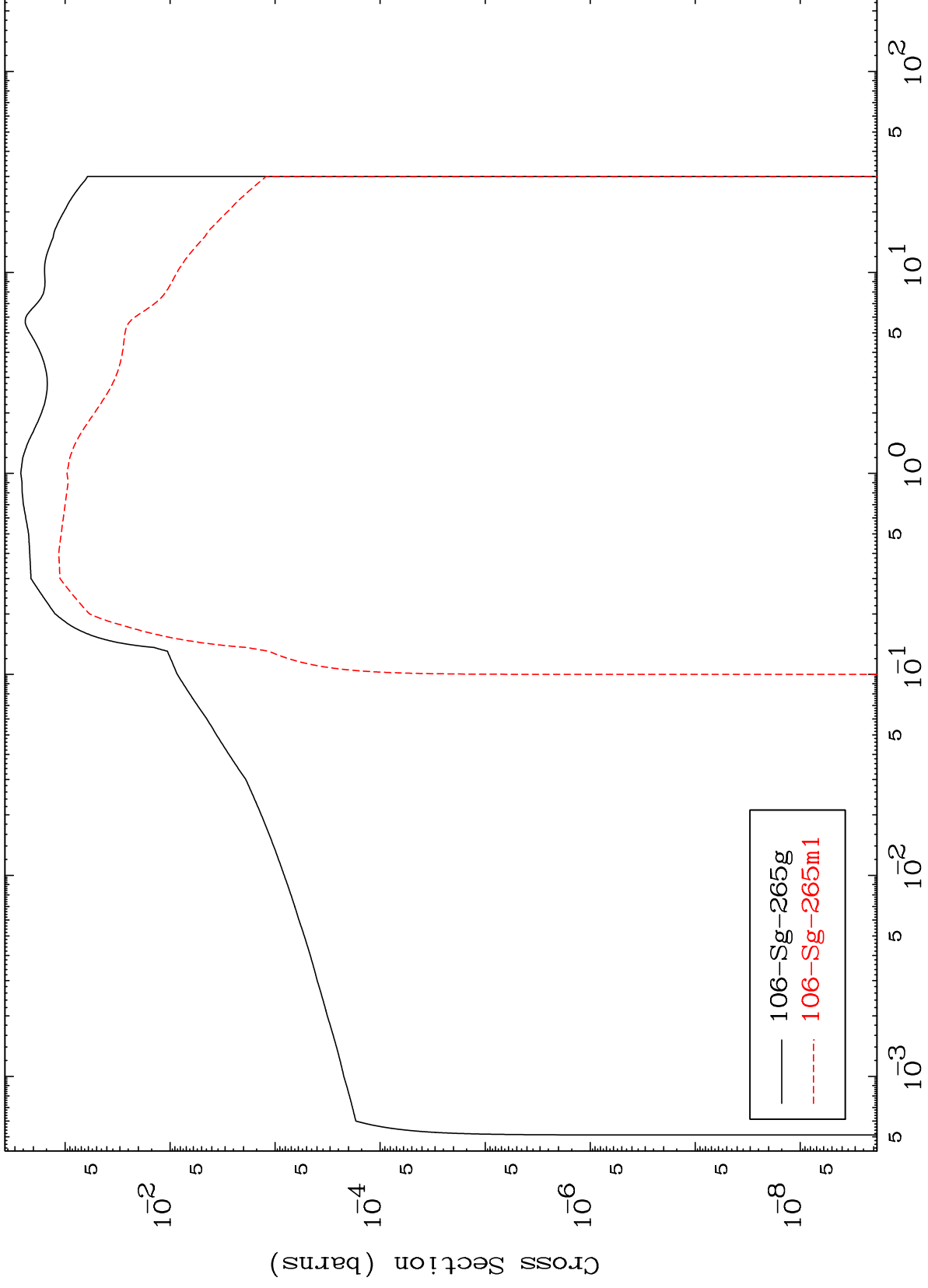
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Inelastic  
Radionuclide Production Cross Section



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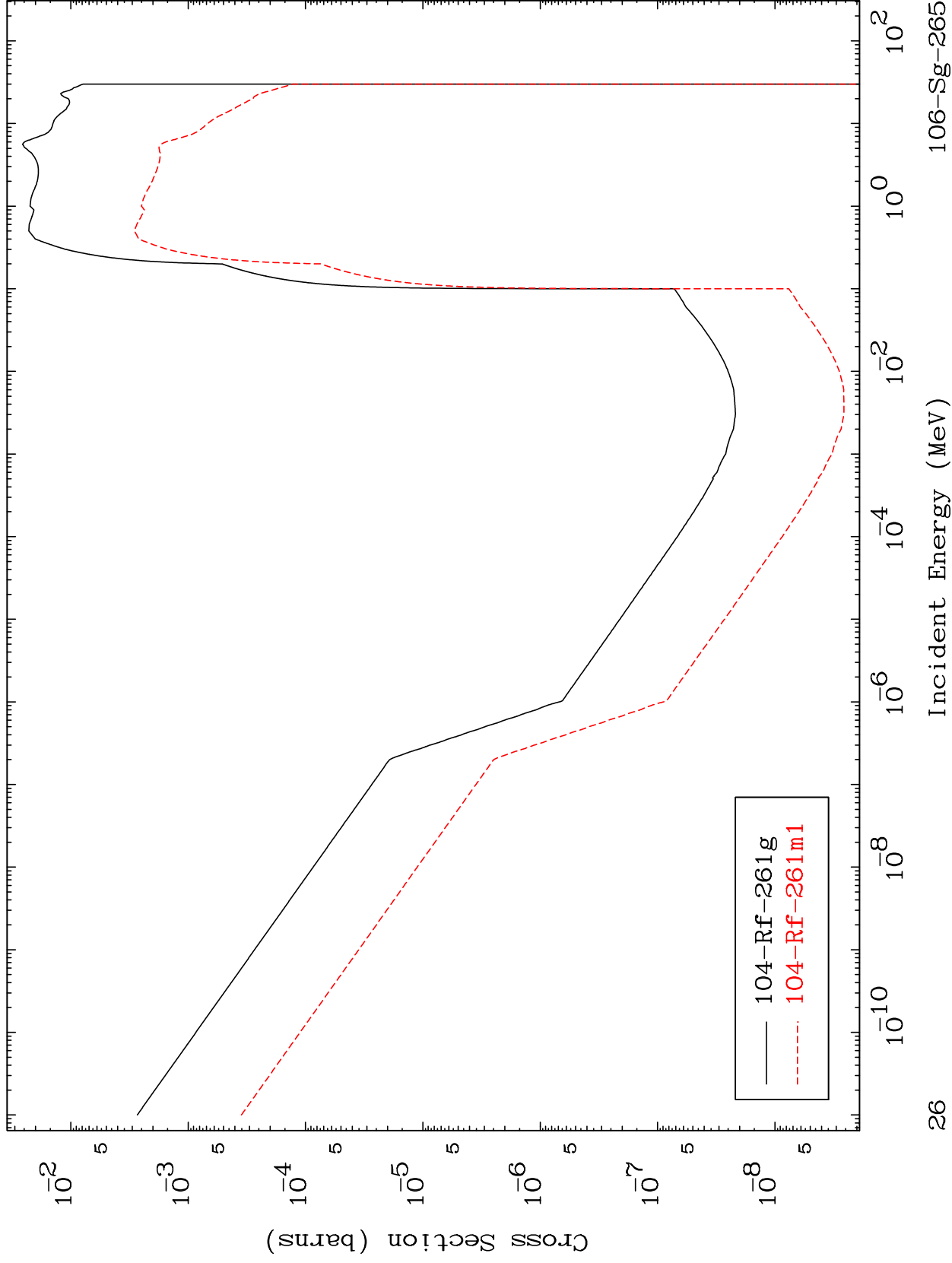
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$(n, n') \alpha$

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Radionuclide Production Cross Section



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MAT 665

(n,n') t,2 $\alpha$

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Radionuclide Production Cross Section

