

Program EVALPLOT
(Version 2015-2)

by

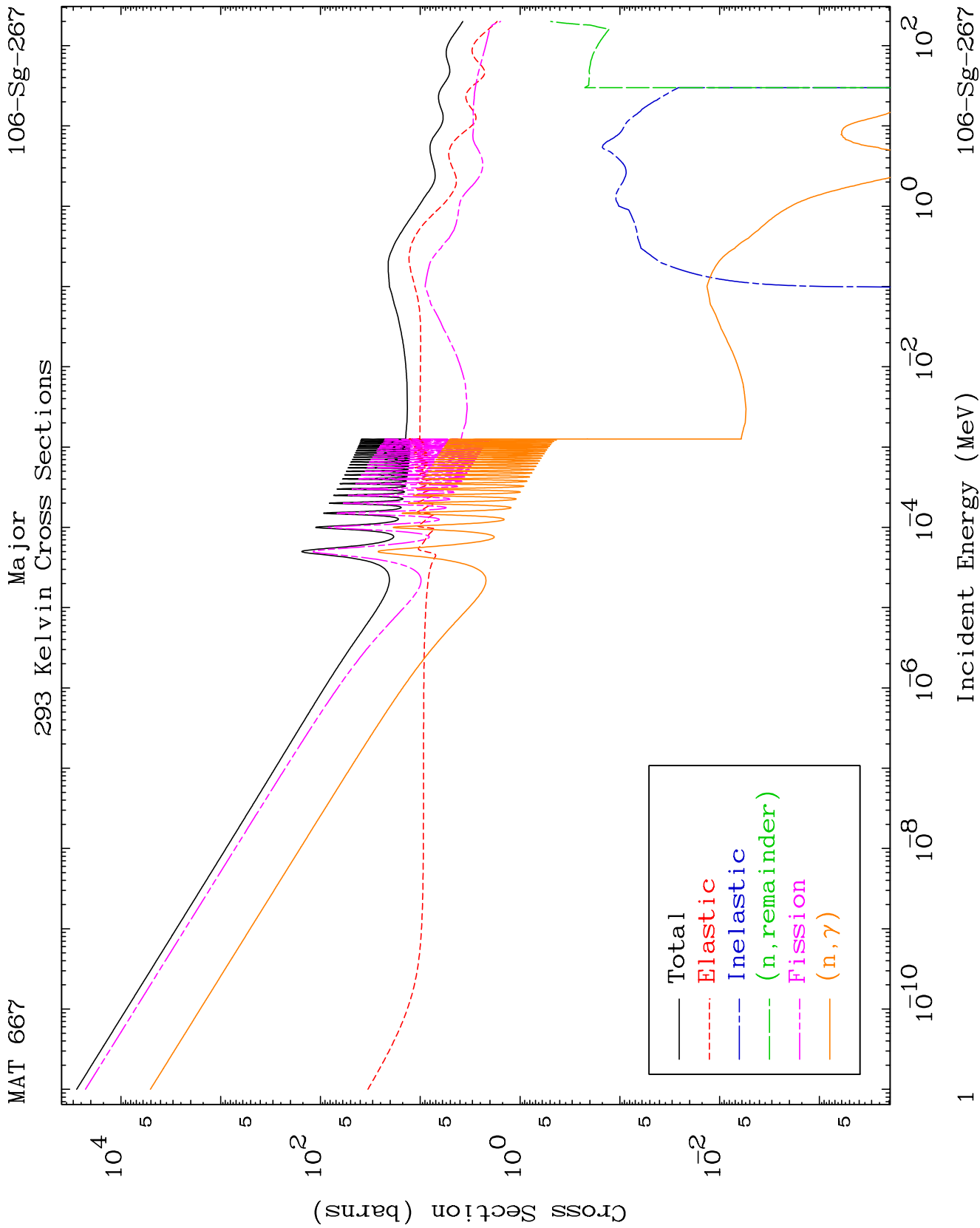
Dermott E. Cullen
(Present Contact Information)

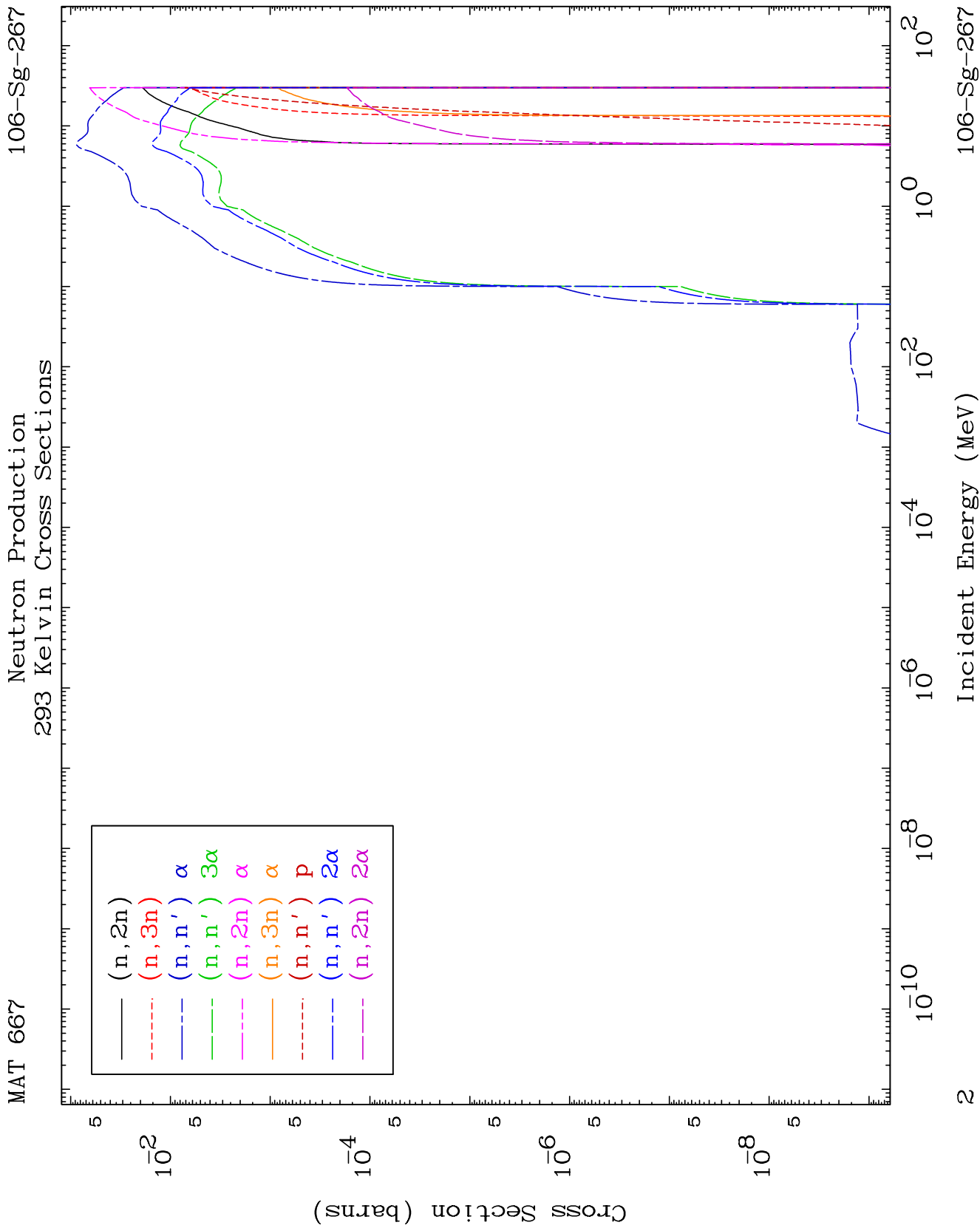
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Press Mouse Button to Start

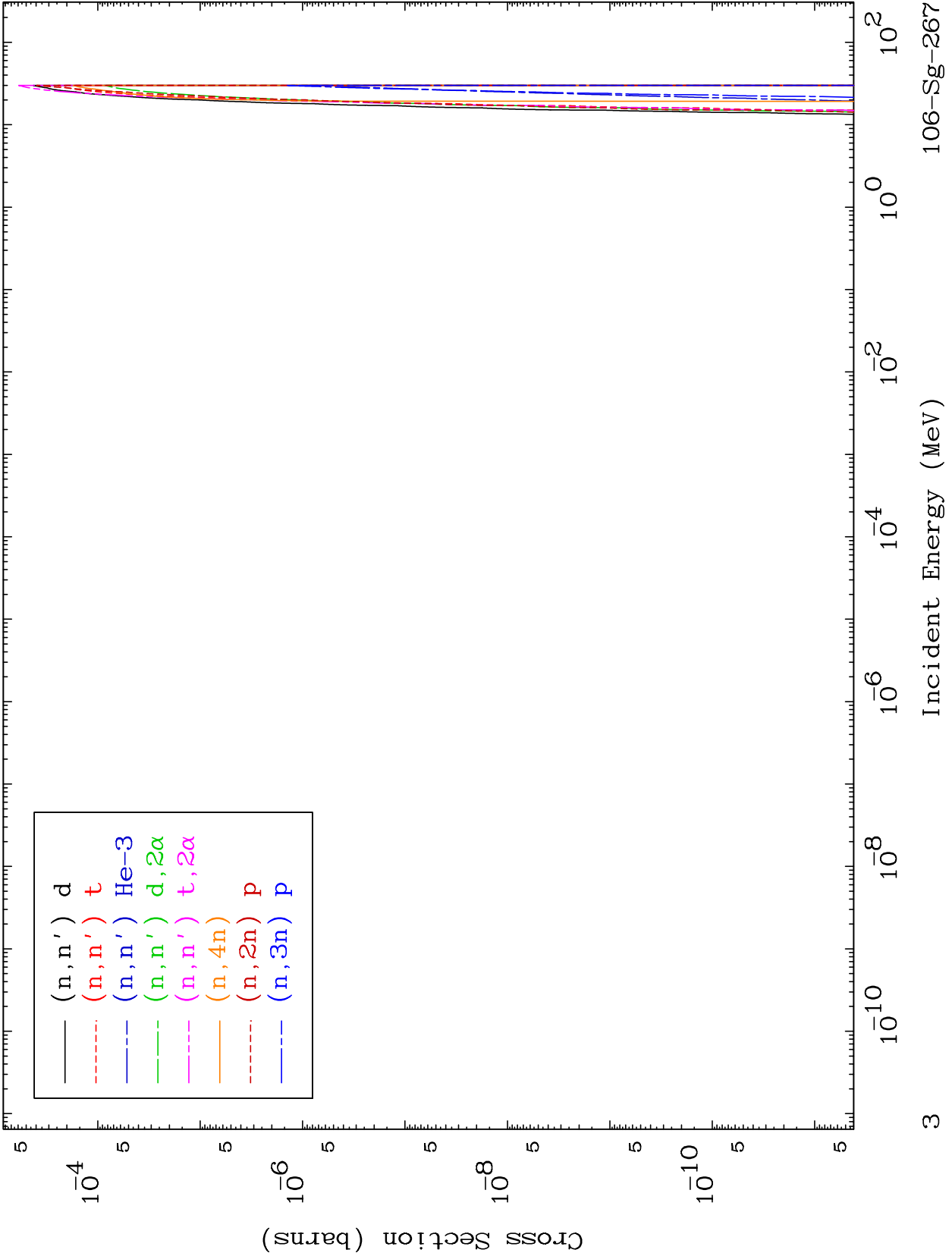




MAT 667

Neutron Production
293 Kelvin Cross Sections

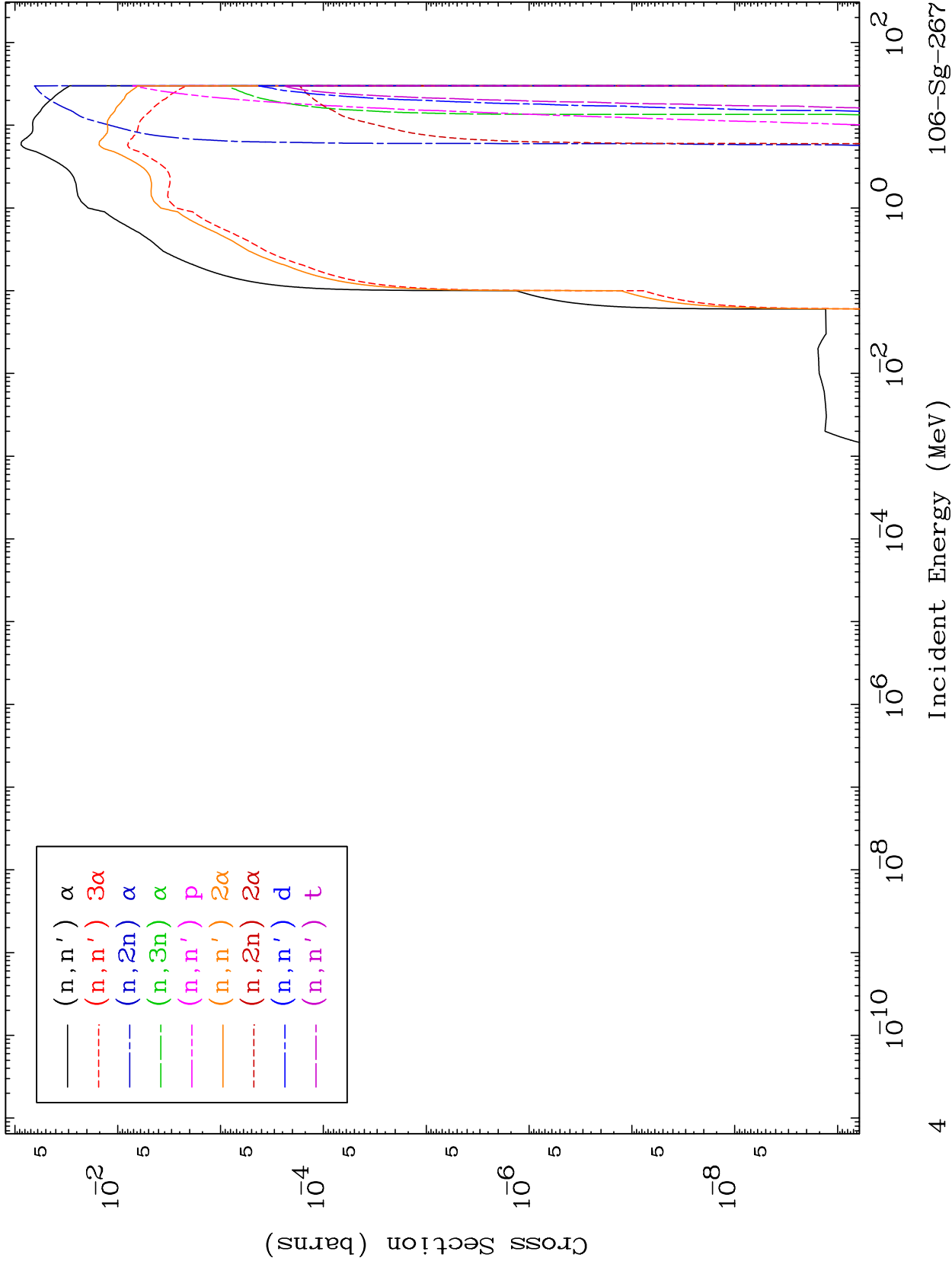
106-Sg-267

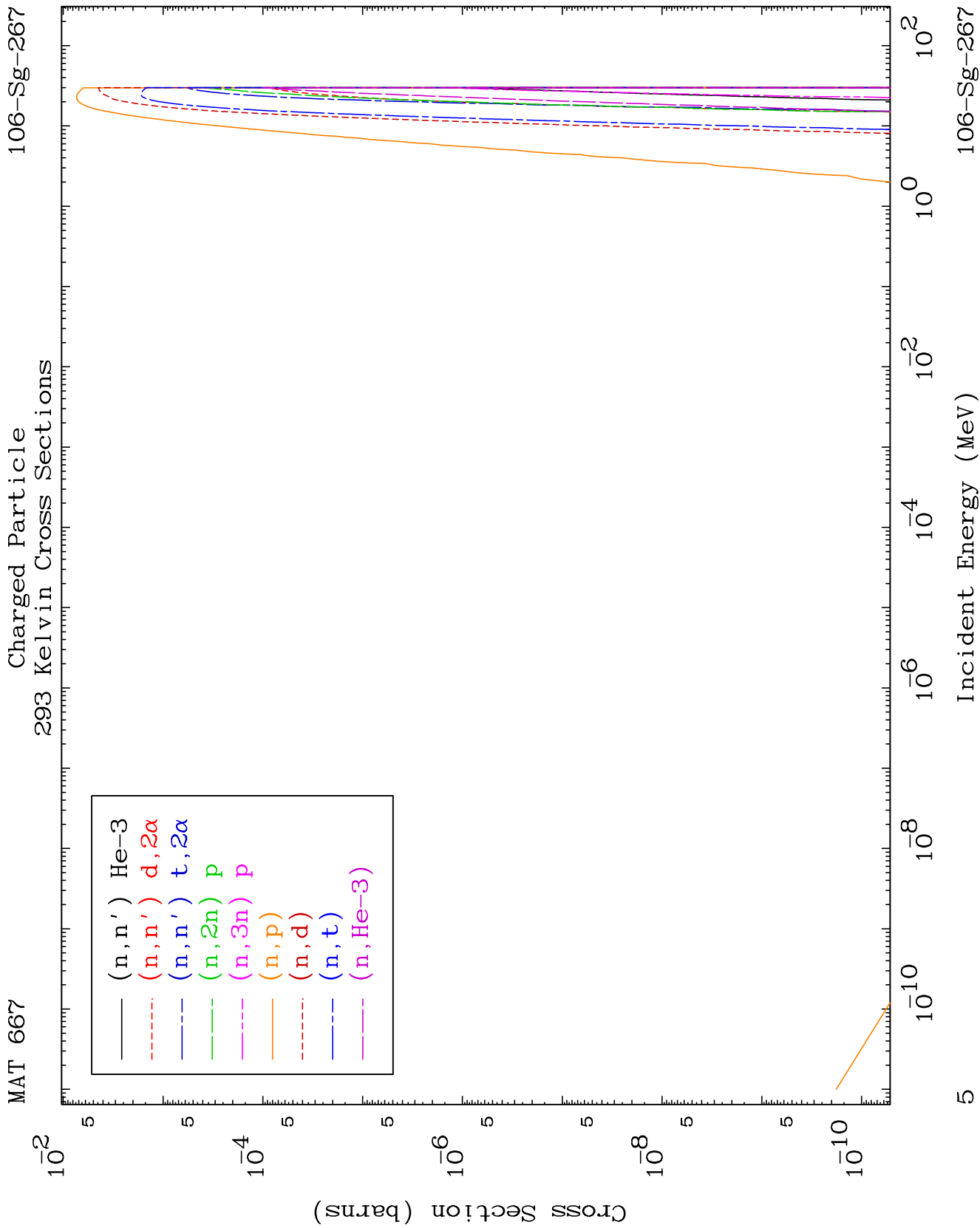


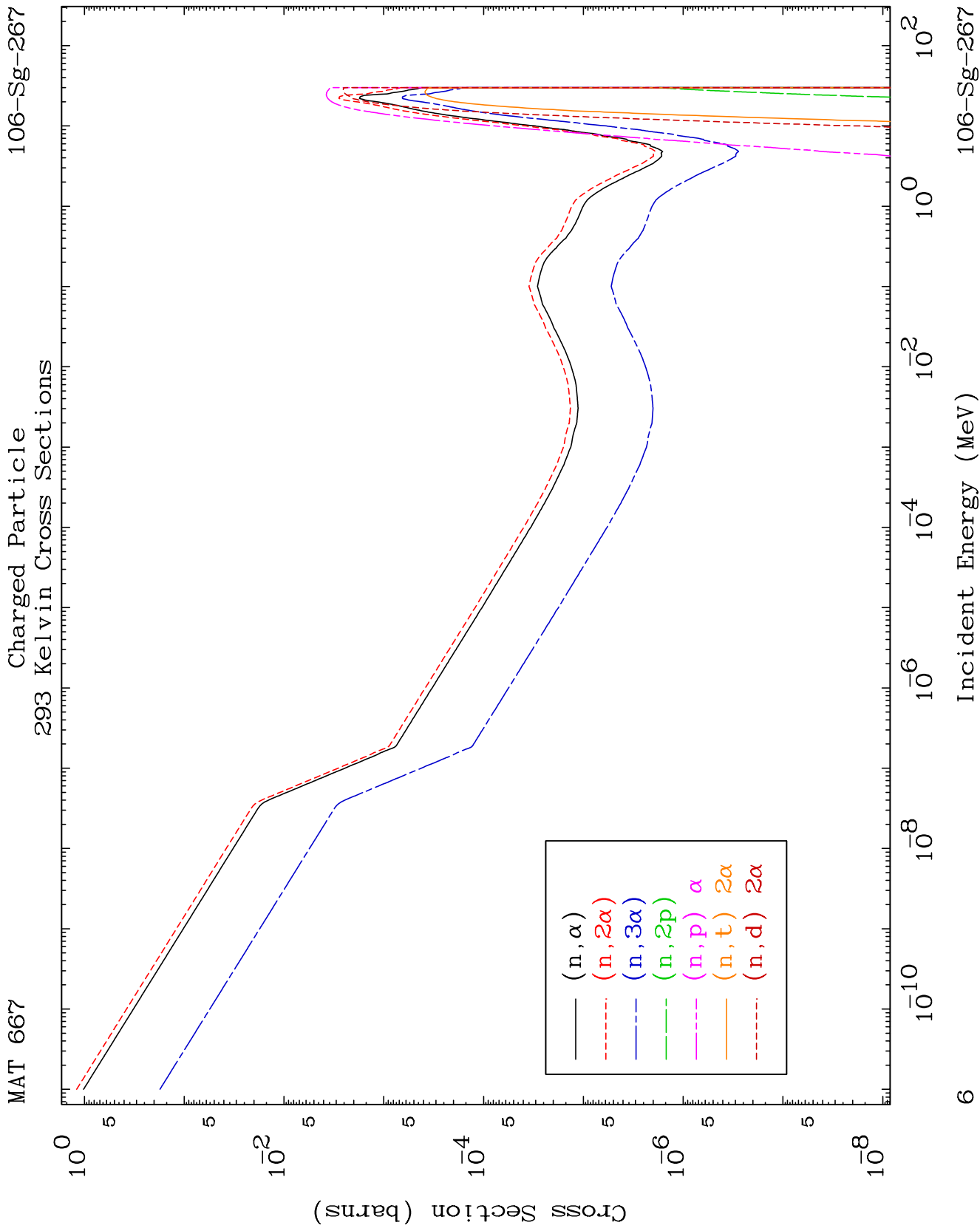
MAT 667

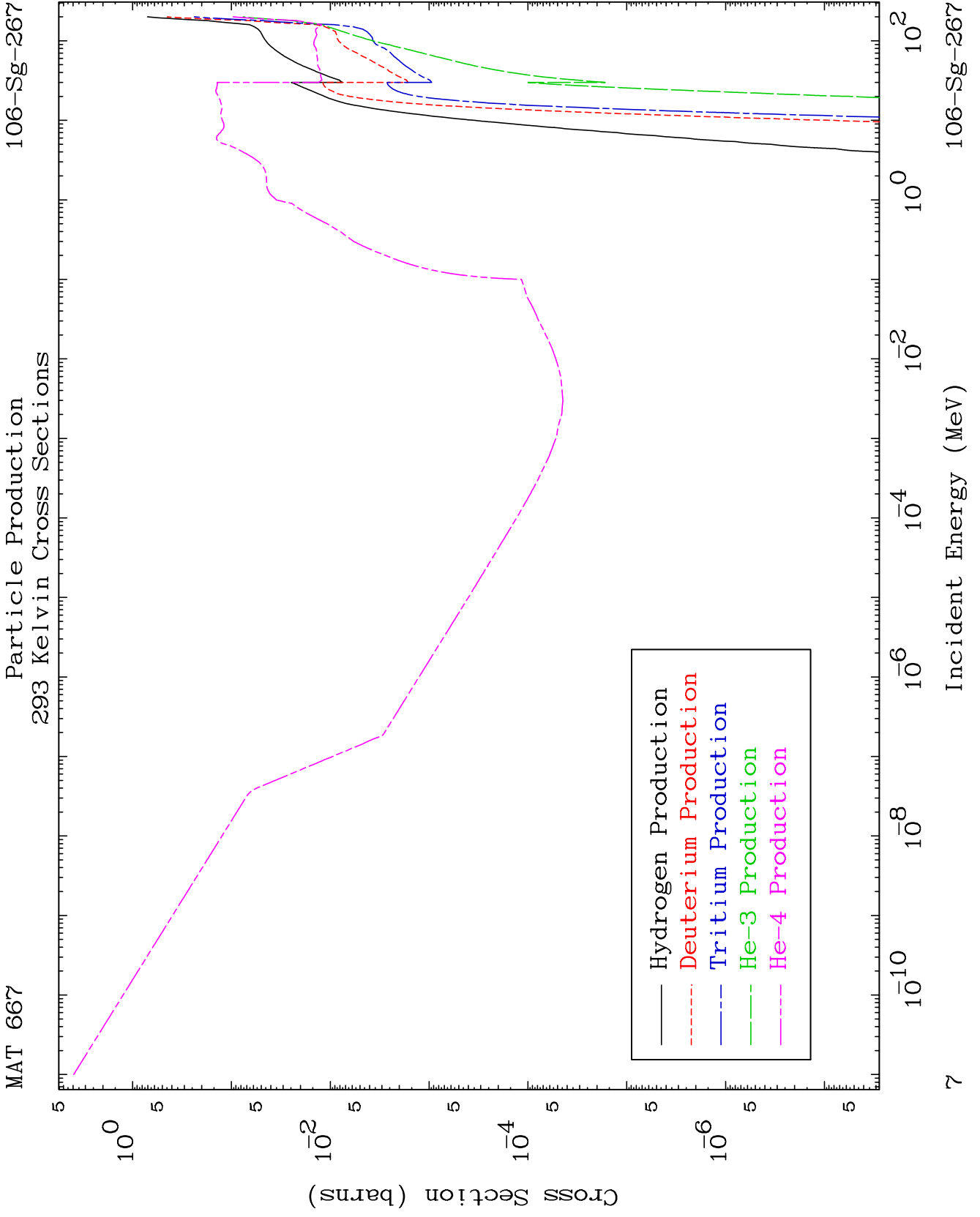
Charged Particle
293 Kelvin Cross Sections

106-Sg-267





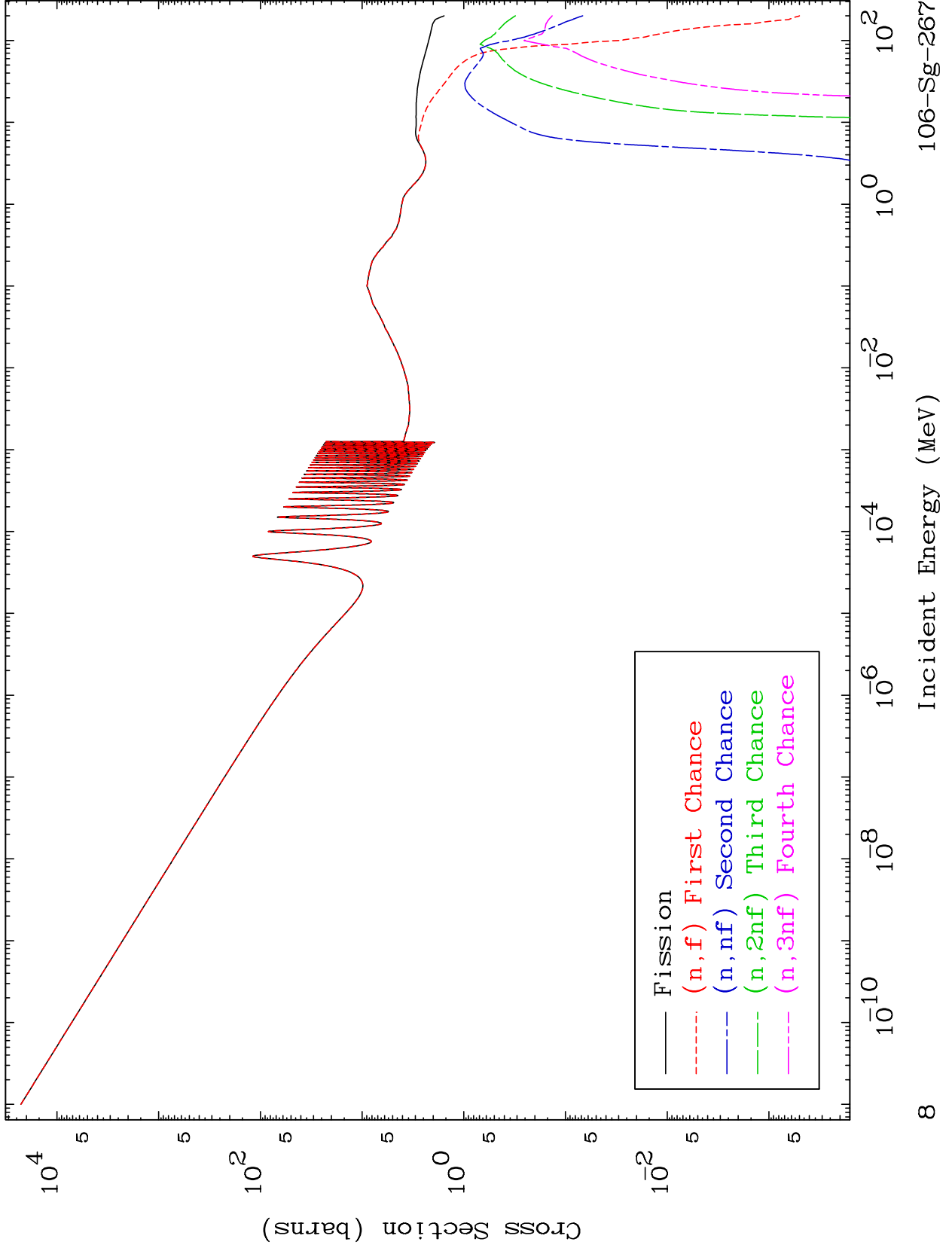




MAT 667

Fission
293 Kelvin Cross Sections

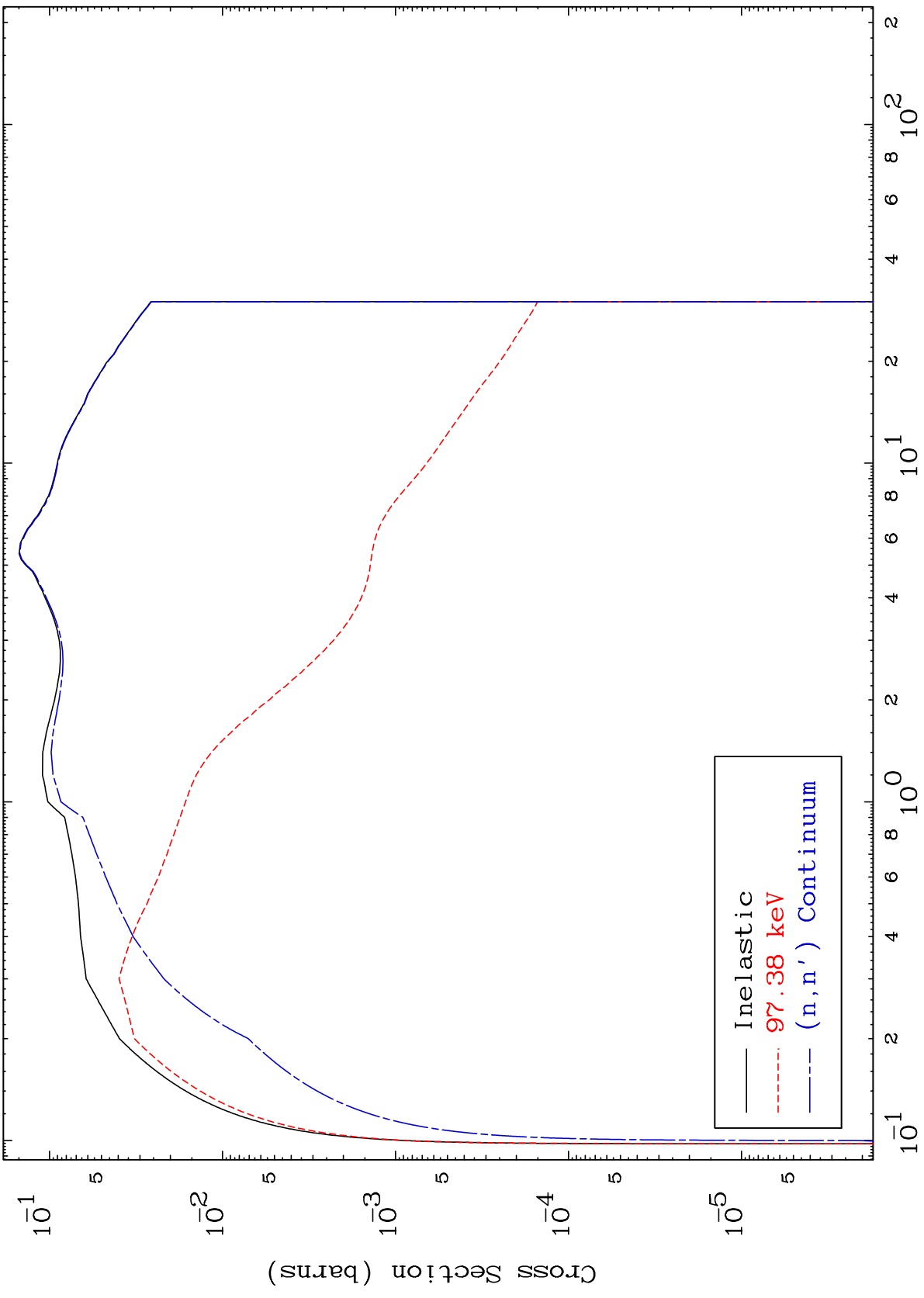
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MAT 667

(n,n') Level
293 Kelvin Cross Sections

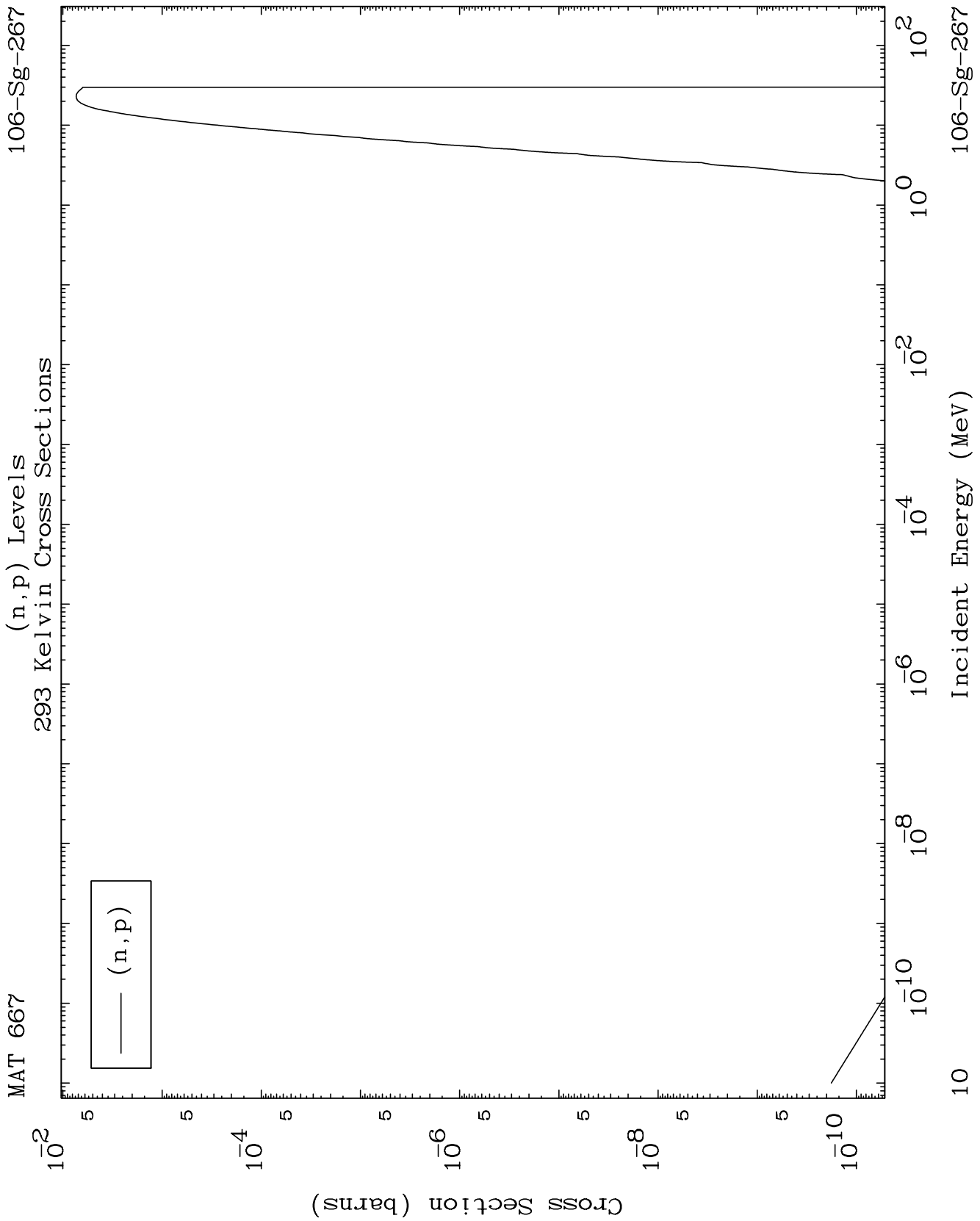
106-Sg-267



Incident Energy (MeV)

106-Sg-267

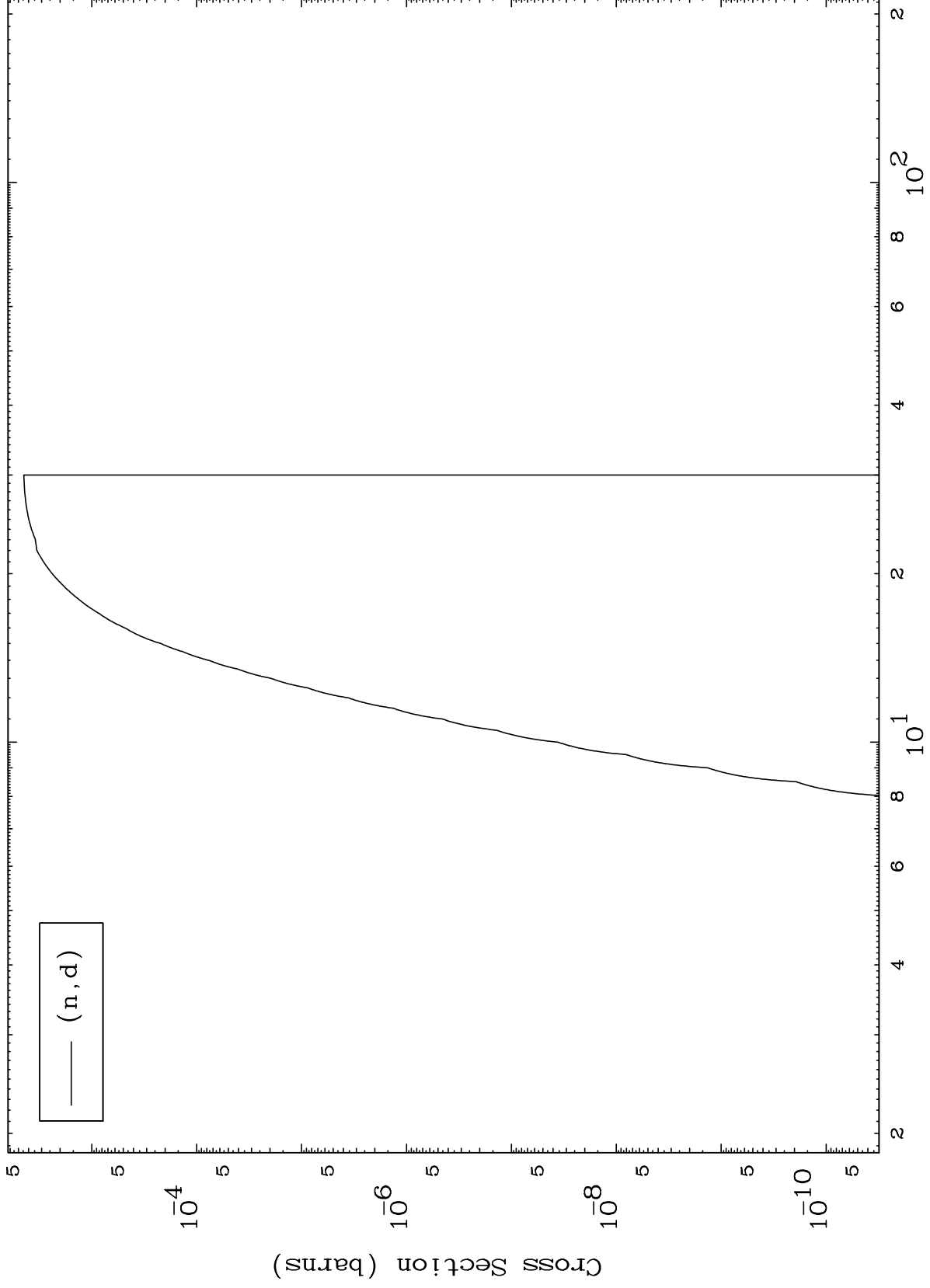
9



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(n,d) Levels
293 Kelvin Cross Sections

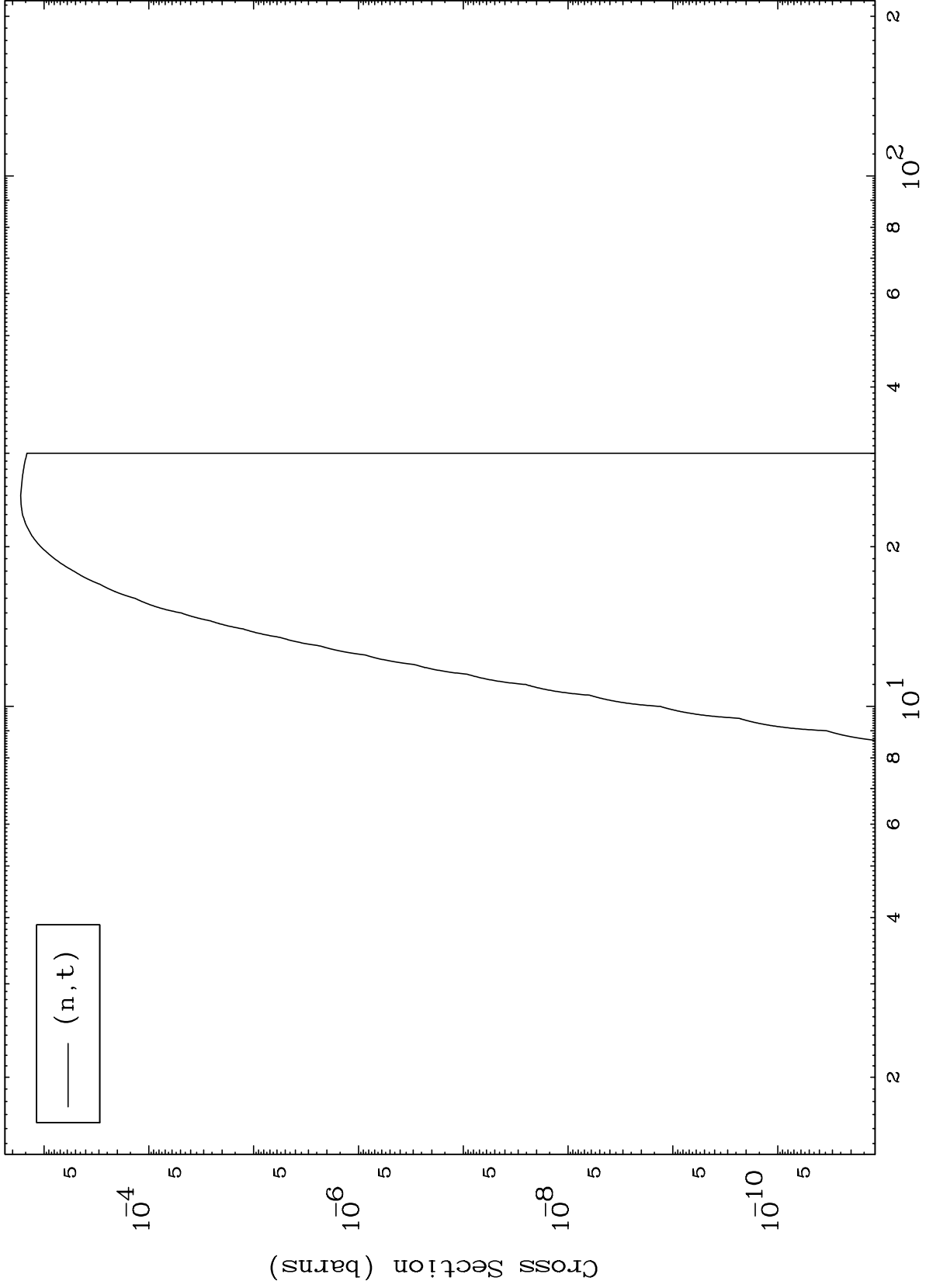
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(n,t) Levels
293 Kelvin Cross Sections

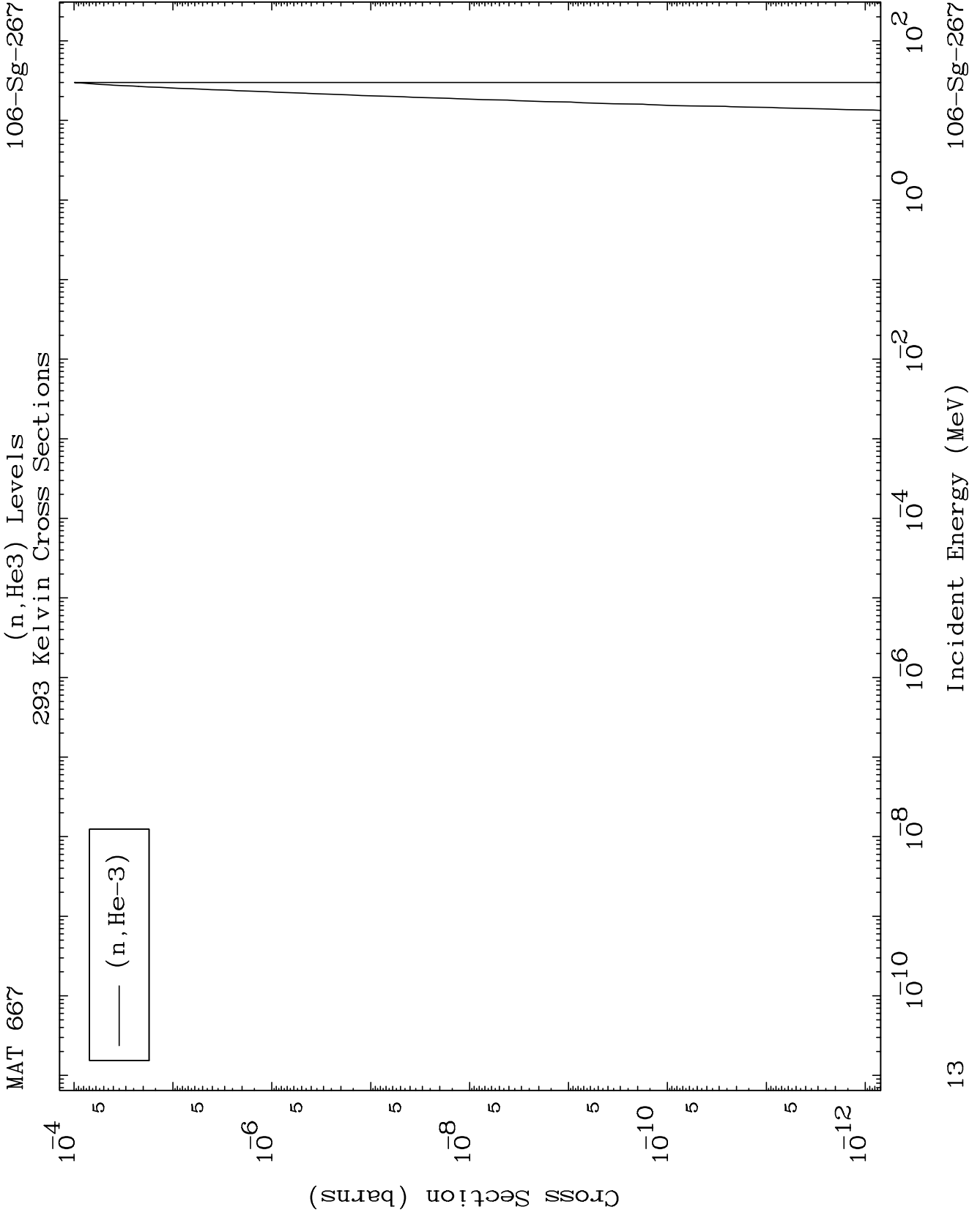
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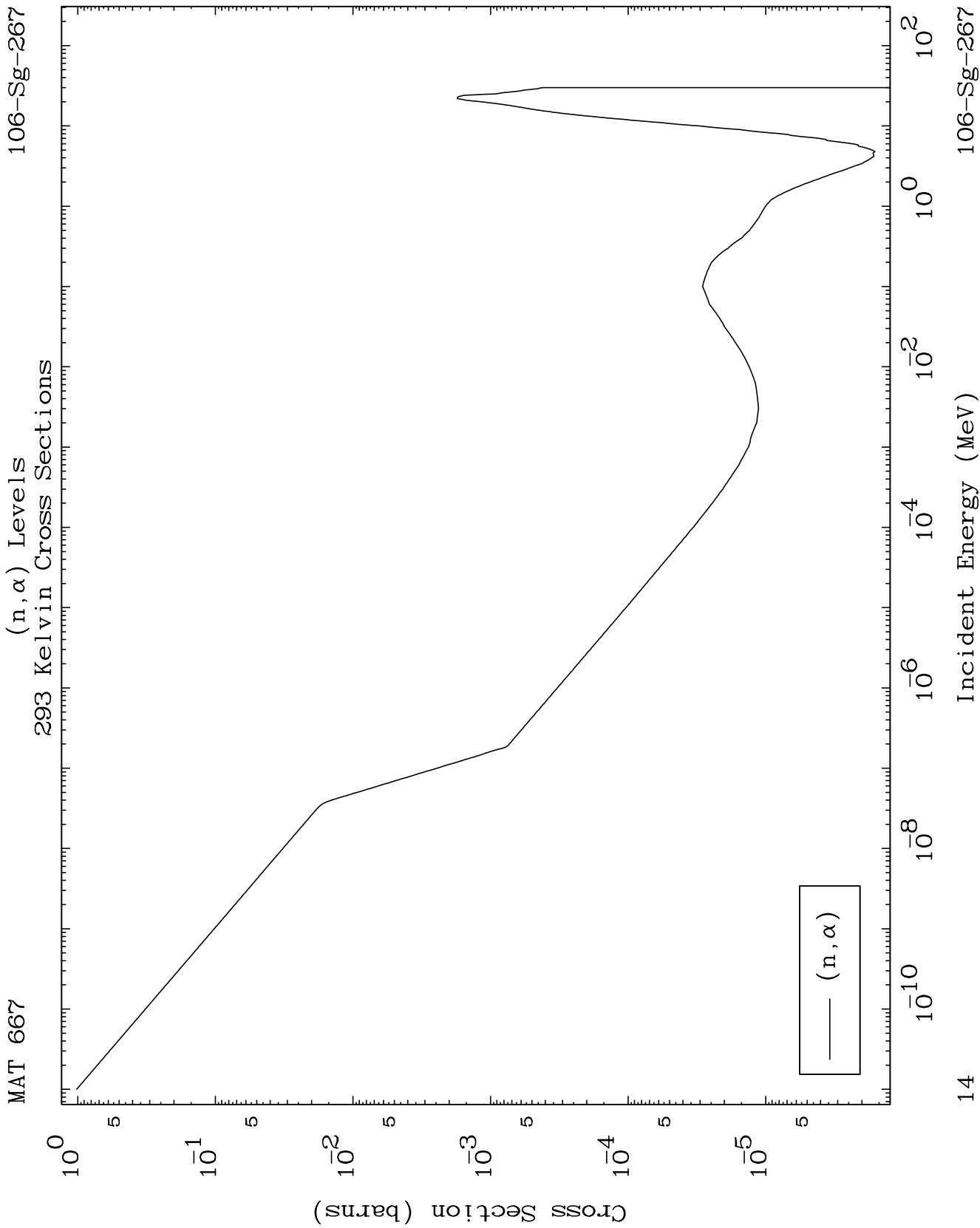


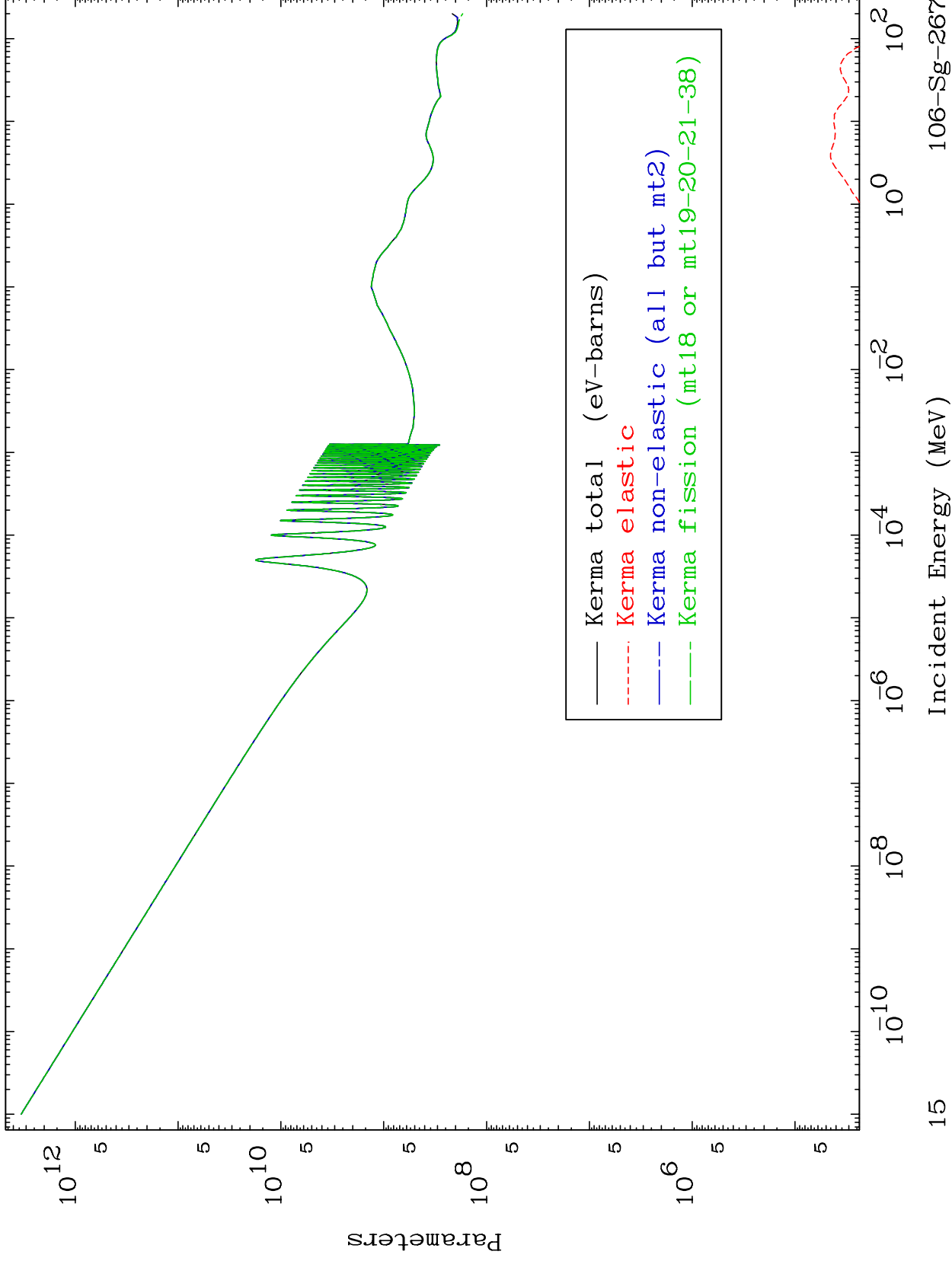
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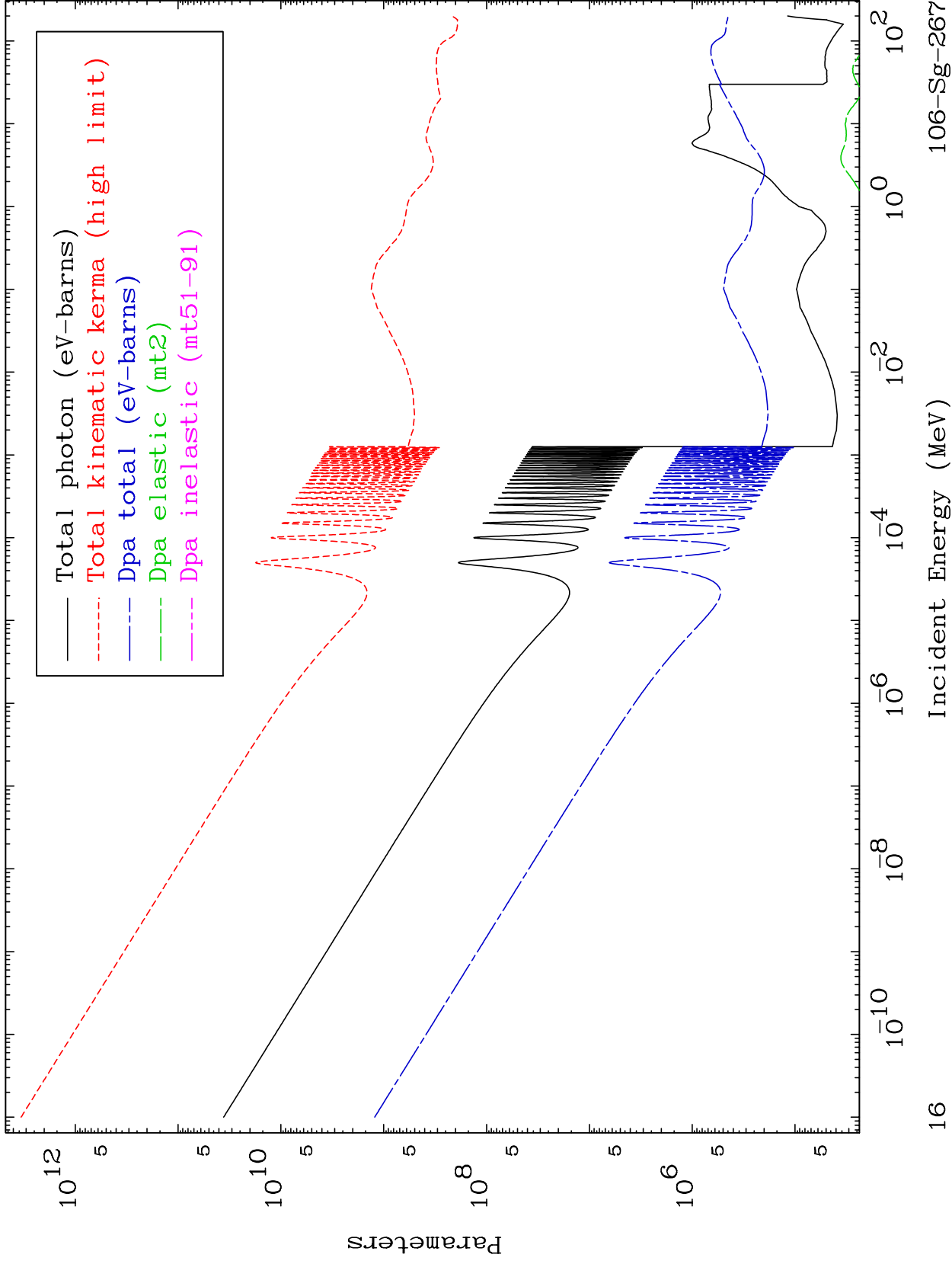
Incident Energy (MeV)

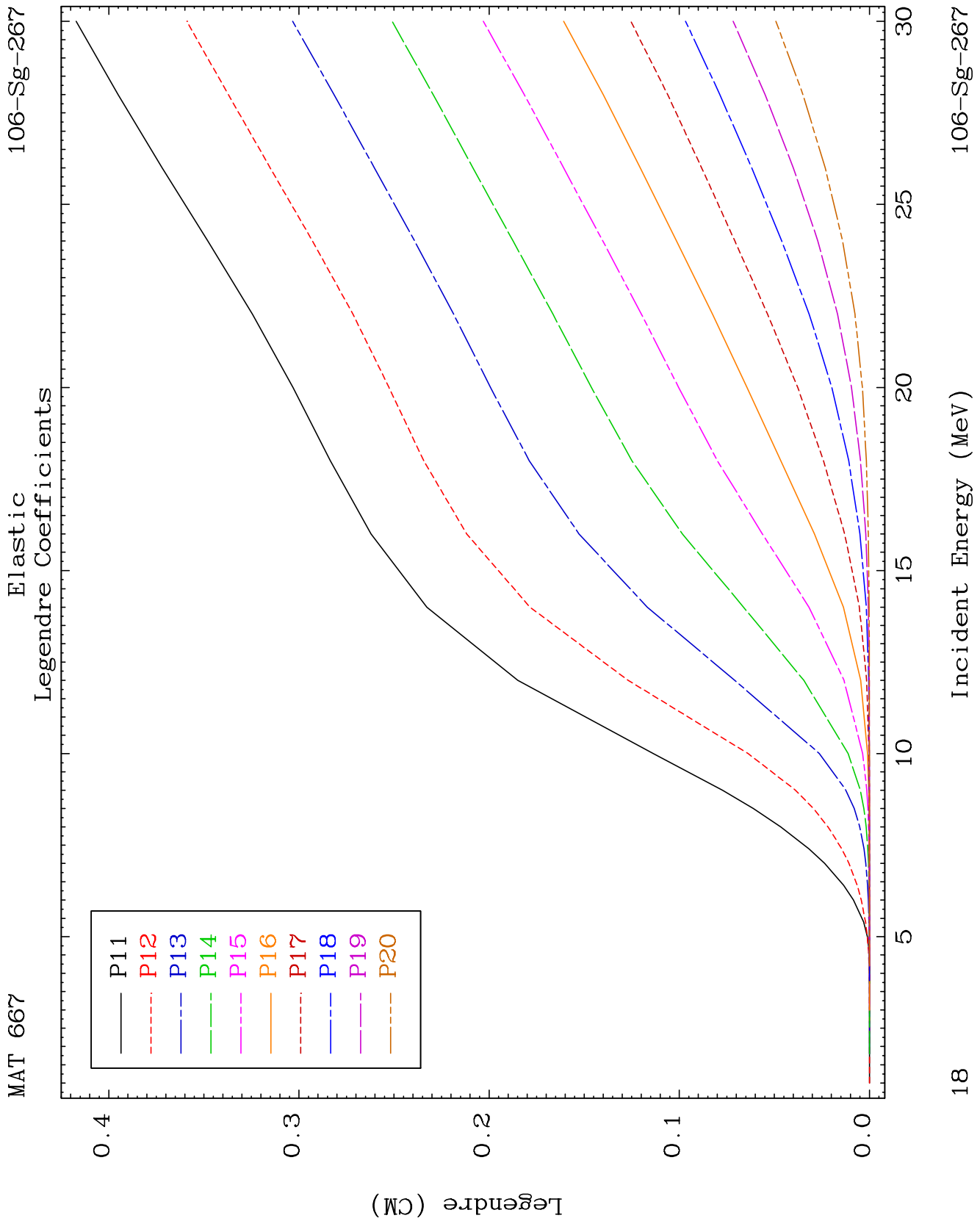
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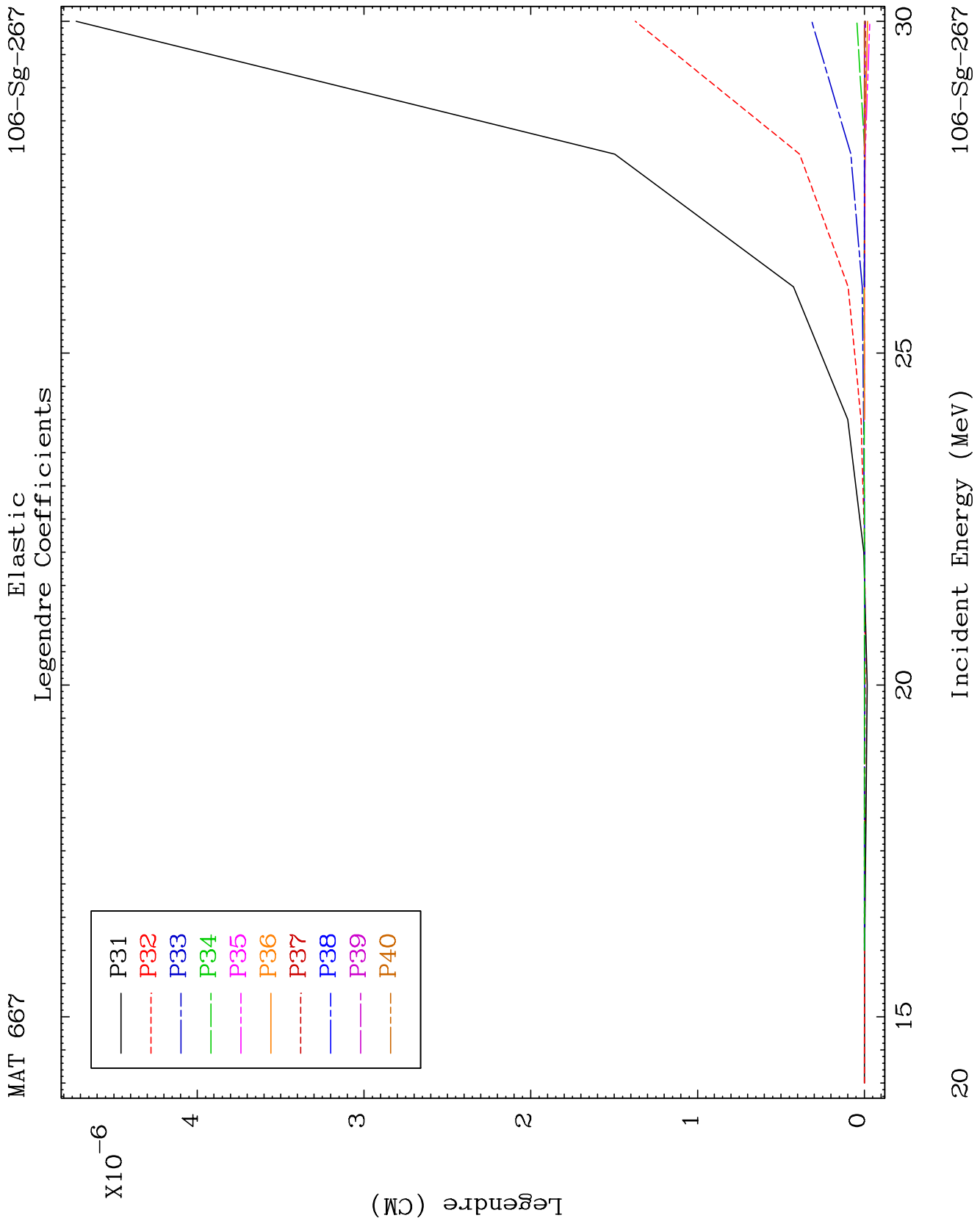




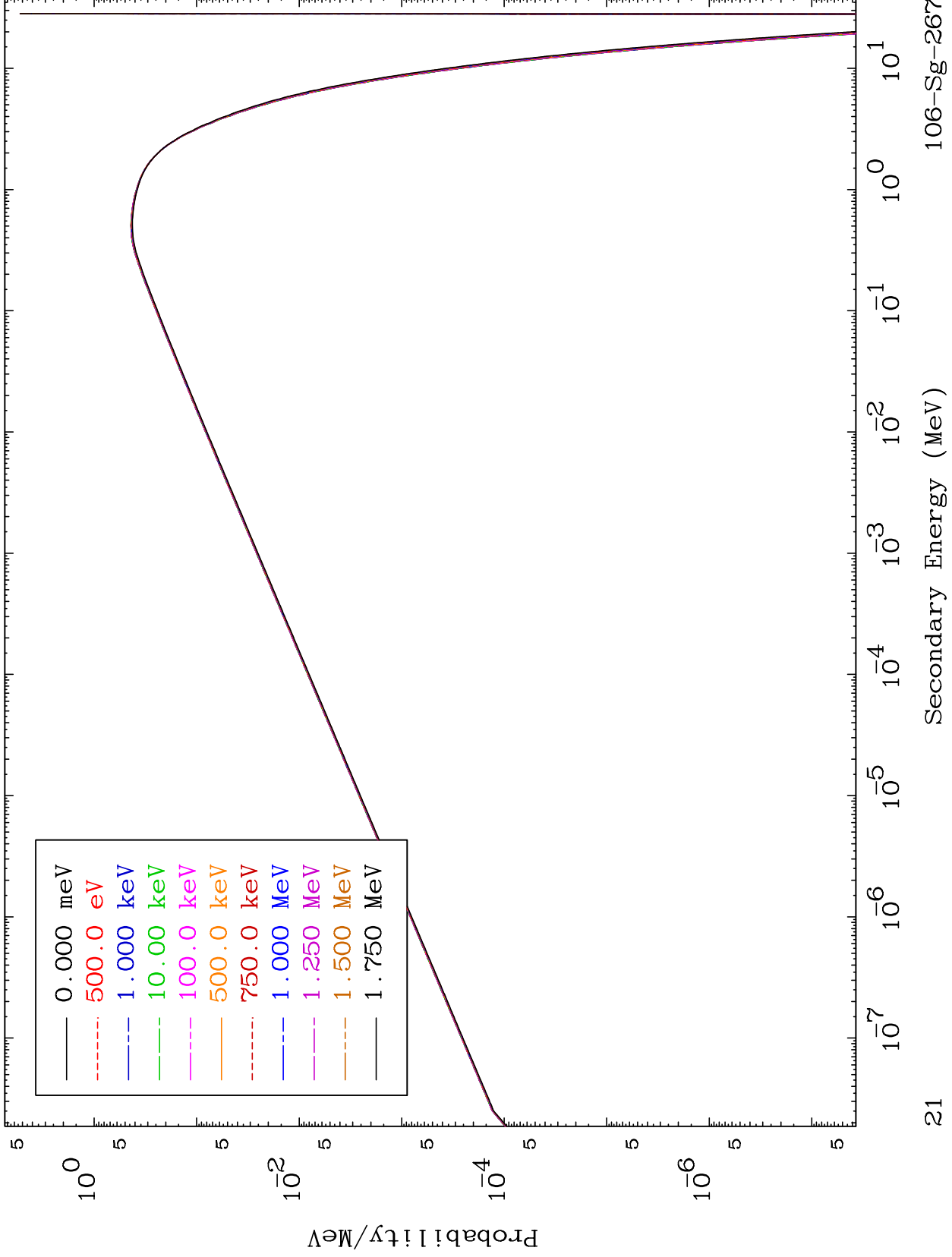




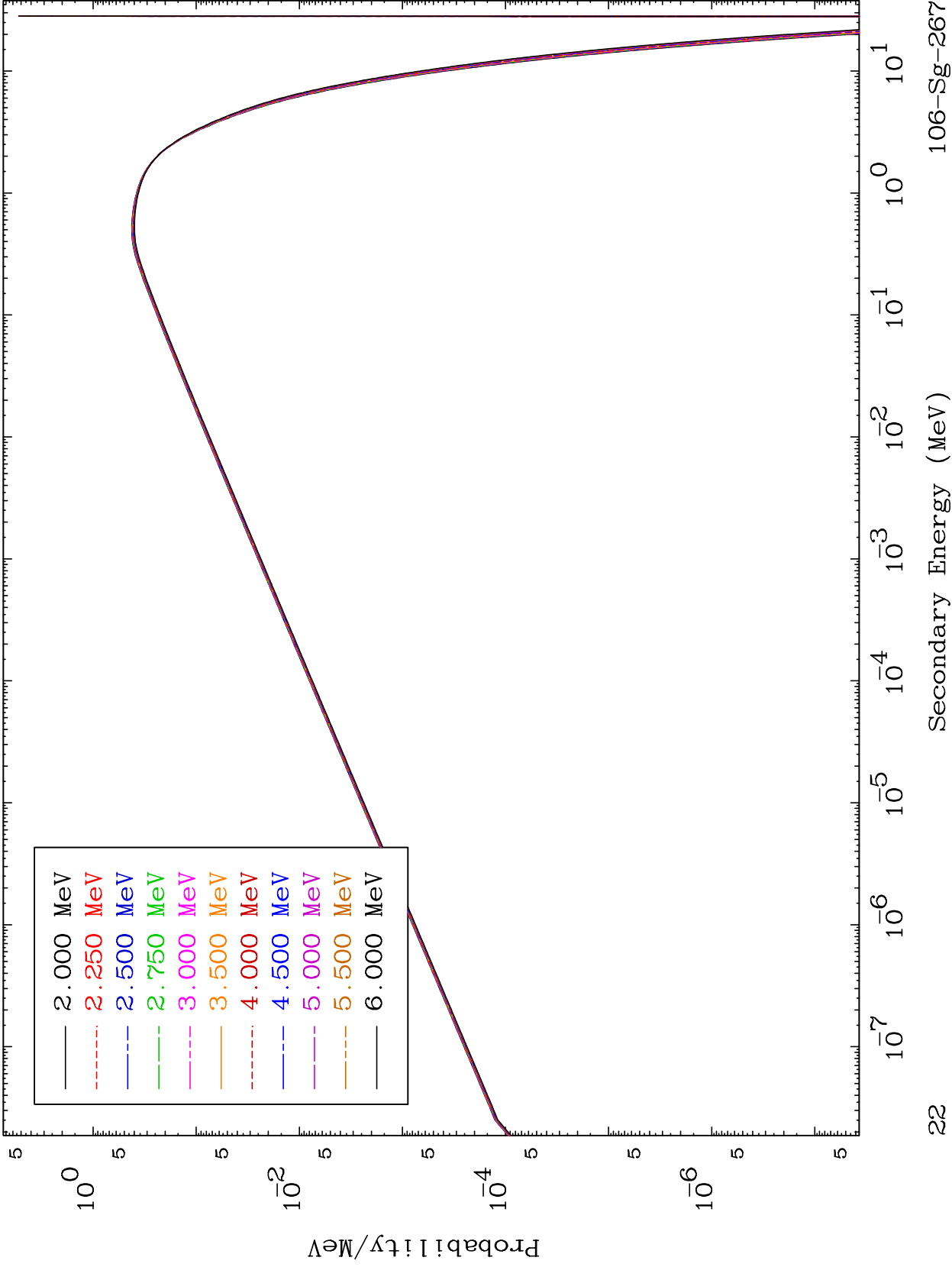




Fission
Energy Distribution



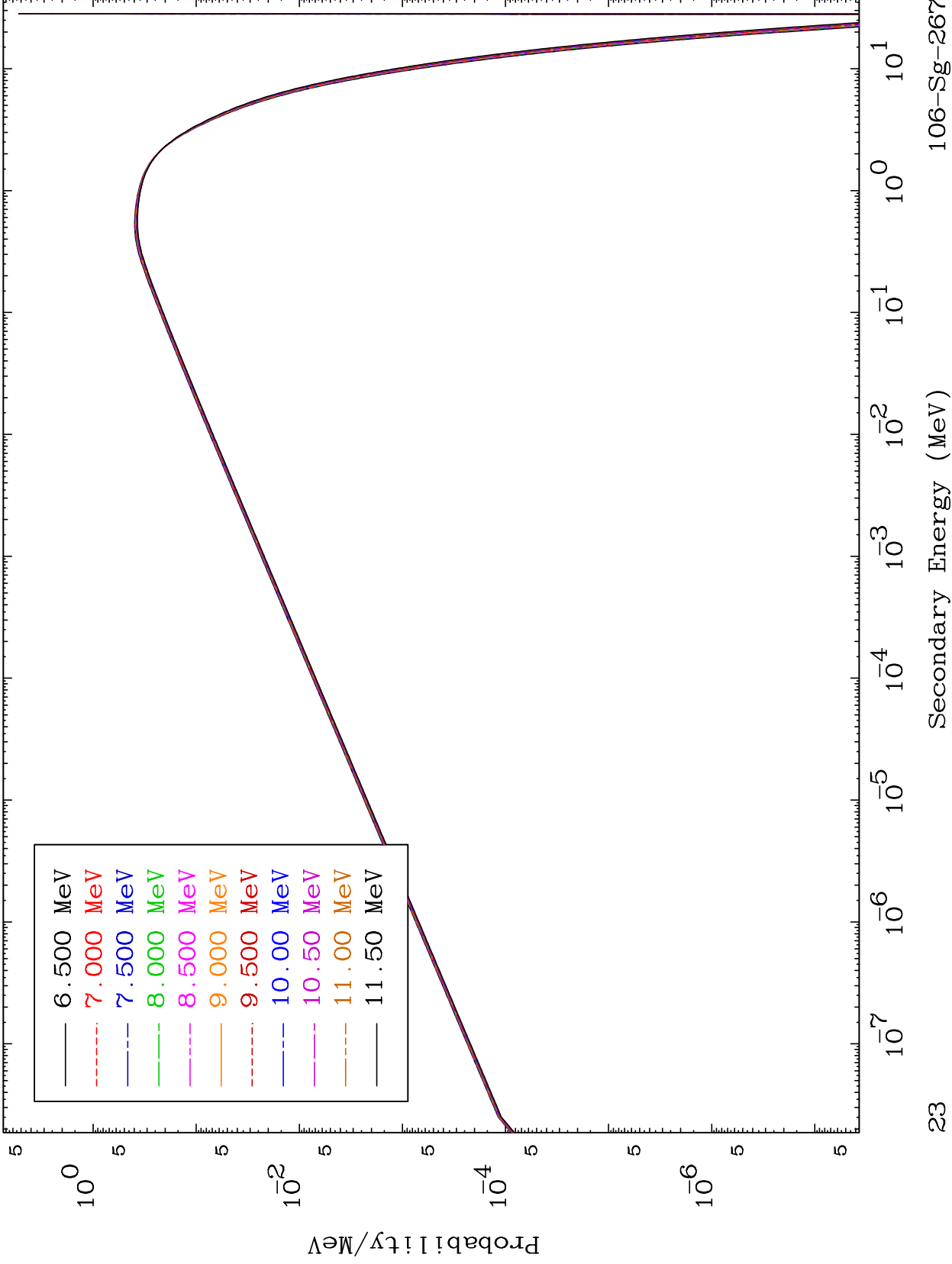
Fission
Energy Distribution



MAT 667

Fission
Energy Distribution

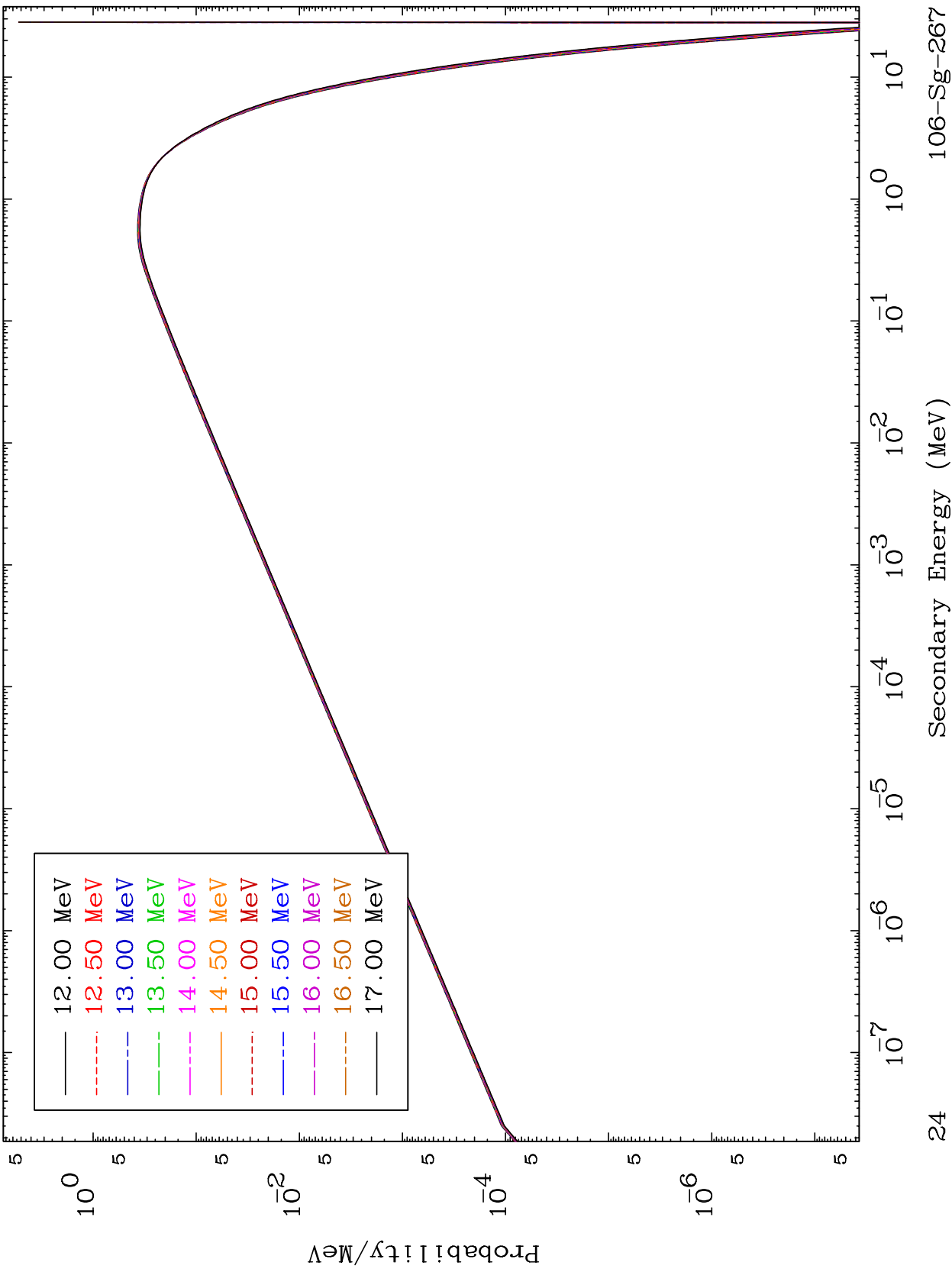
106-Sg-267



MAT 667

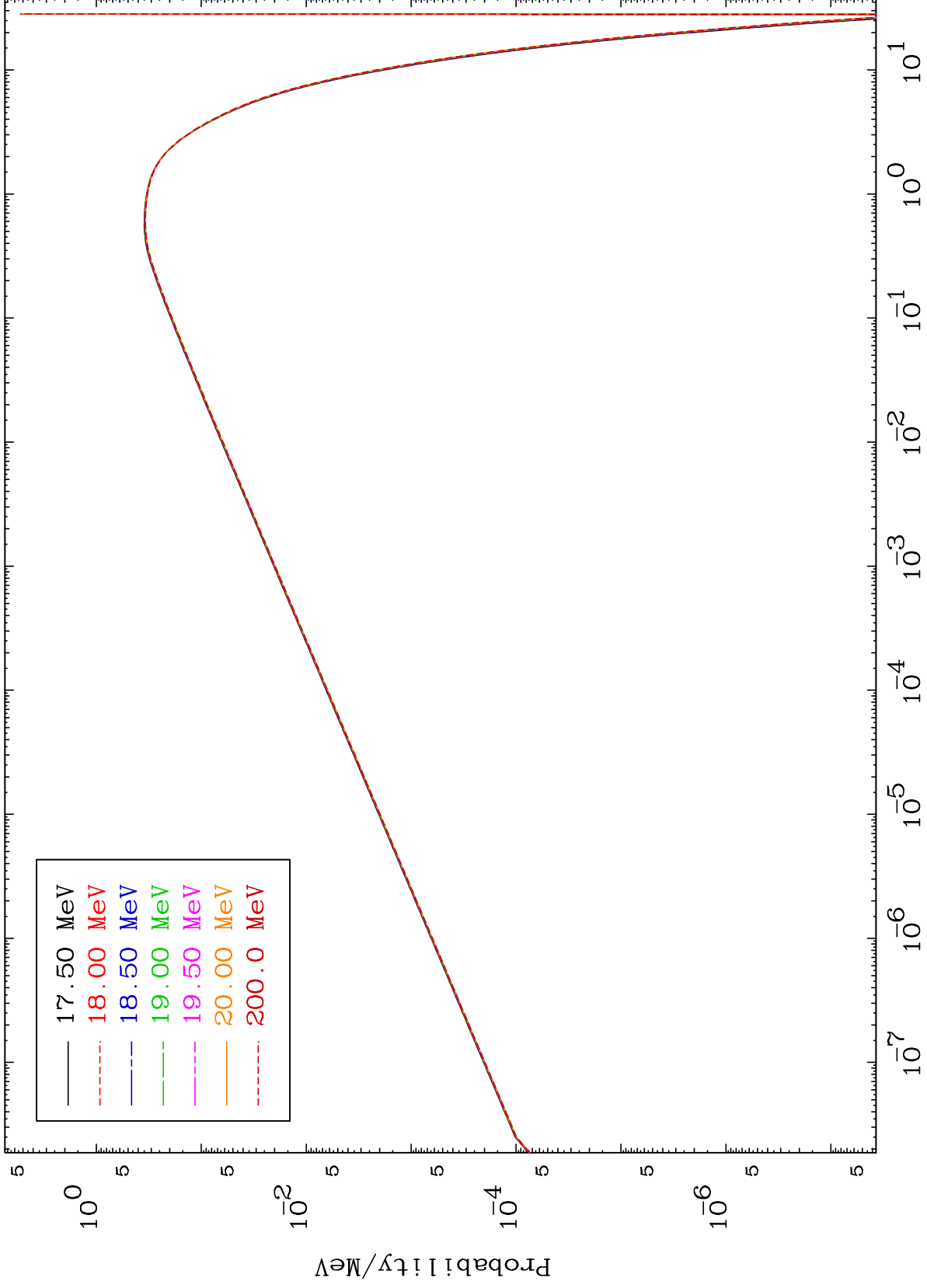
Fission Energy Distribution

106-Sg-267

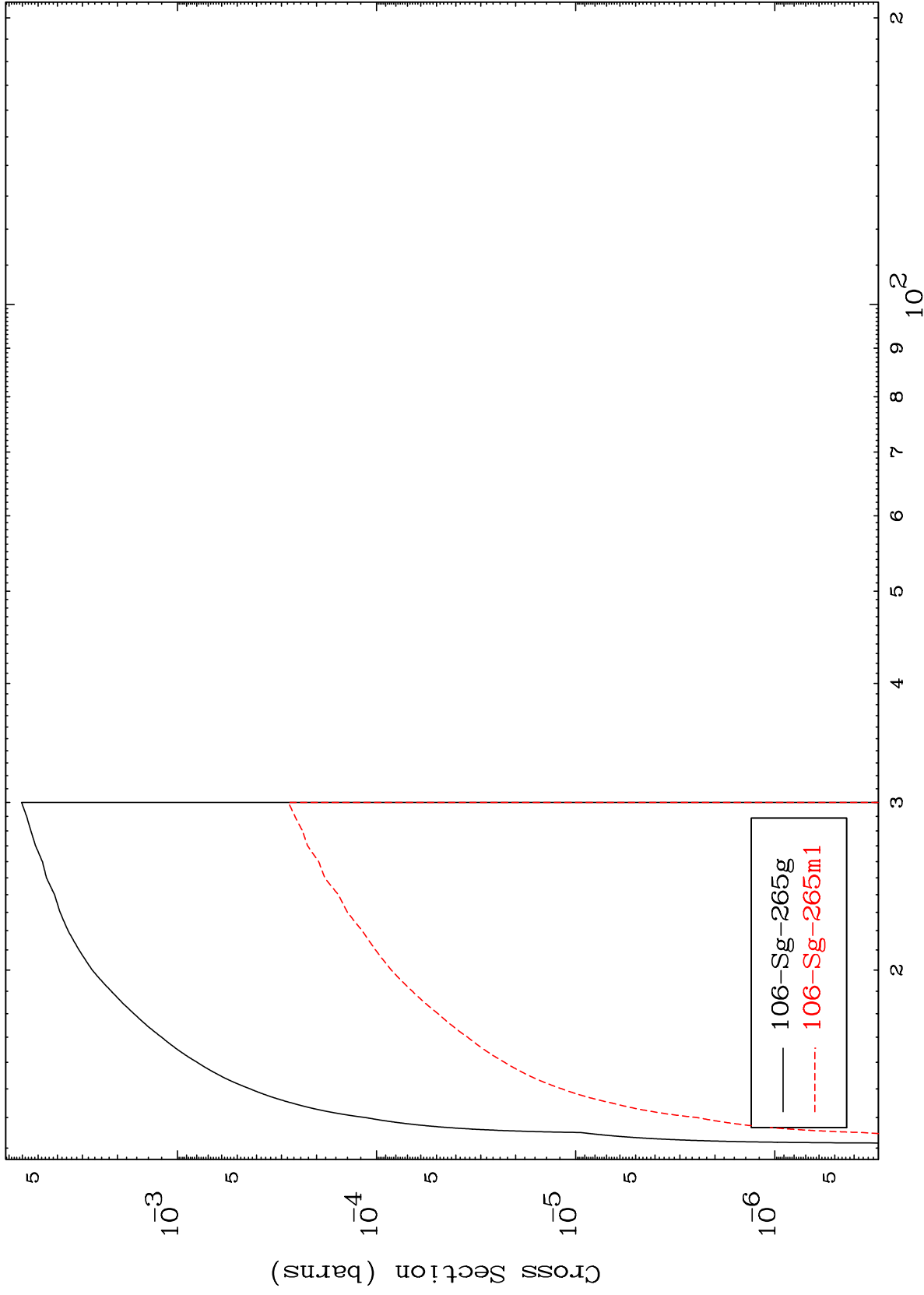


106-Sg-267

Fission
Energy Distribution



(n,3n)
Radionuclide Production Cross Section

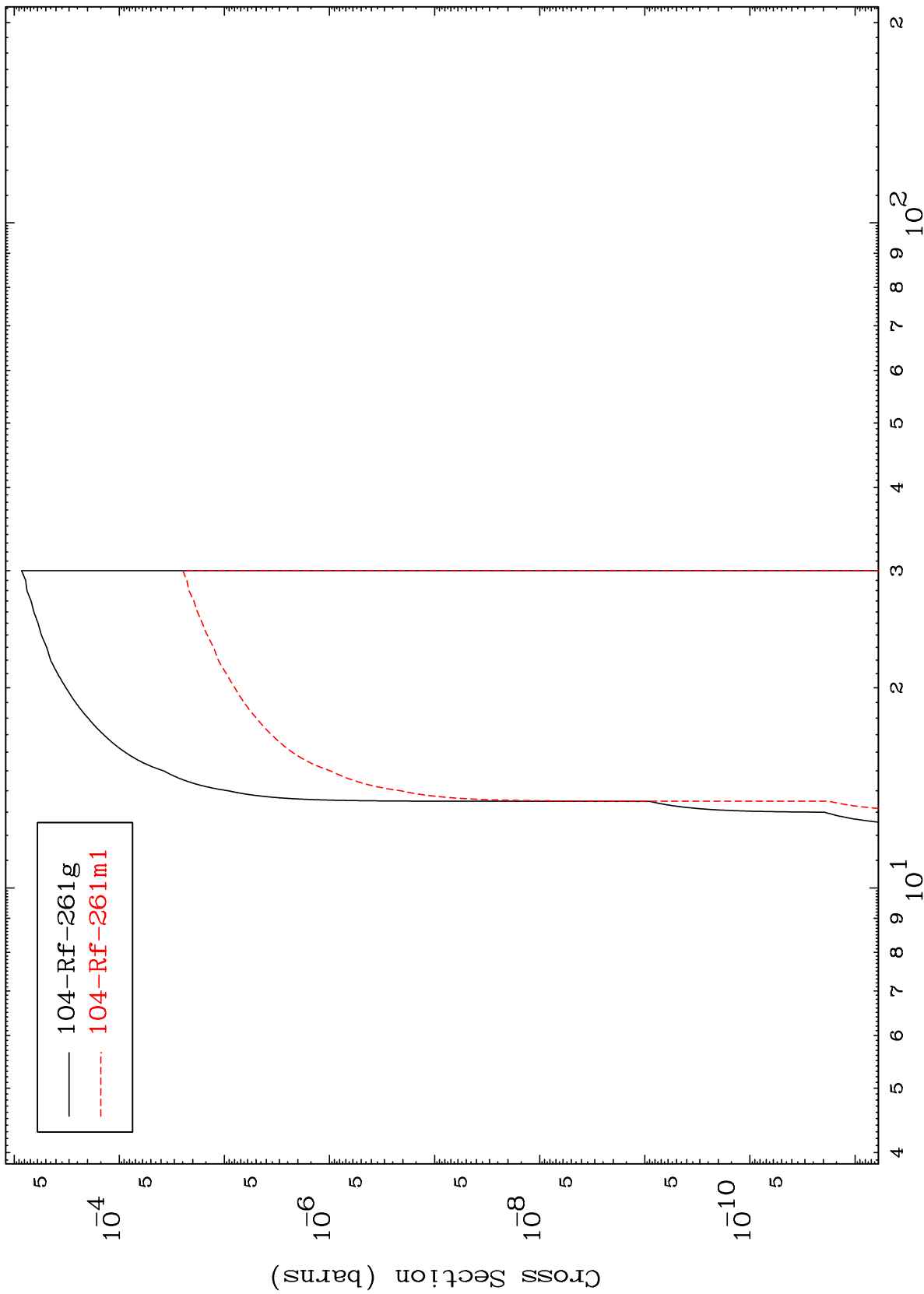


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(n,3n) α

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Radionuclide Production Cross Section

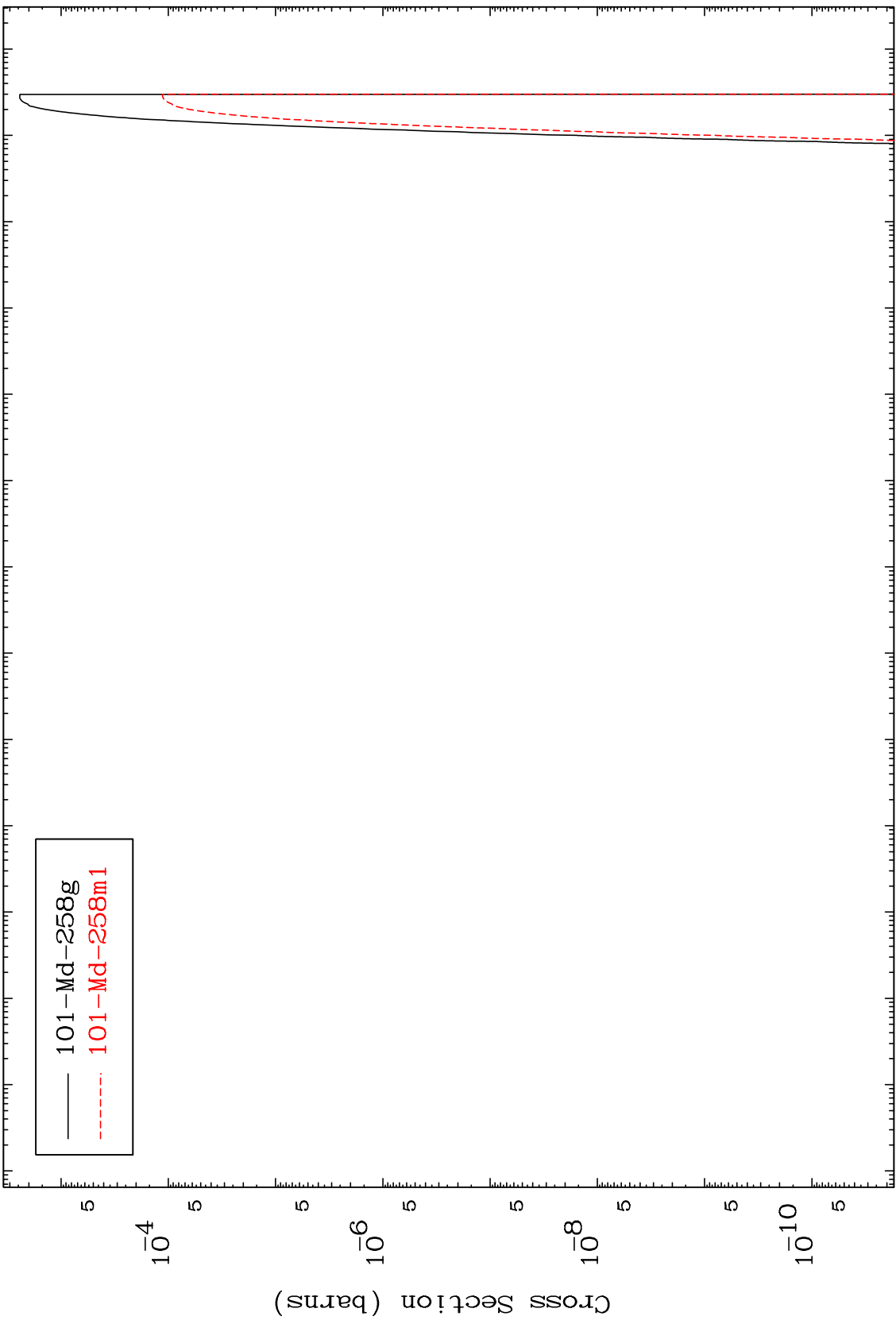
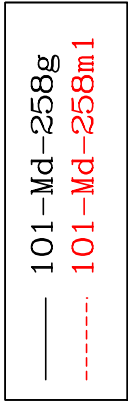


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(n,d) 2 α

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Radionuclide Production Cross Section



106-Sg-267

10⁰

10⁻²

10⁻⁴

10⁻⁶

10⁻⁸

10⁻¹⁰

10²

Incident Energy (MeV)