

Program EVALPLOT
(Version 2015-2)

by

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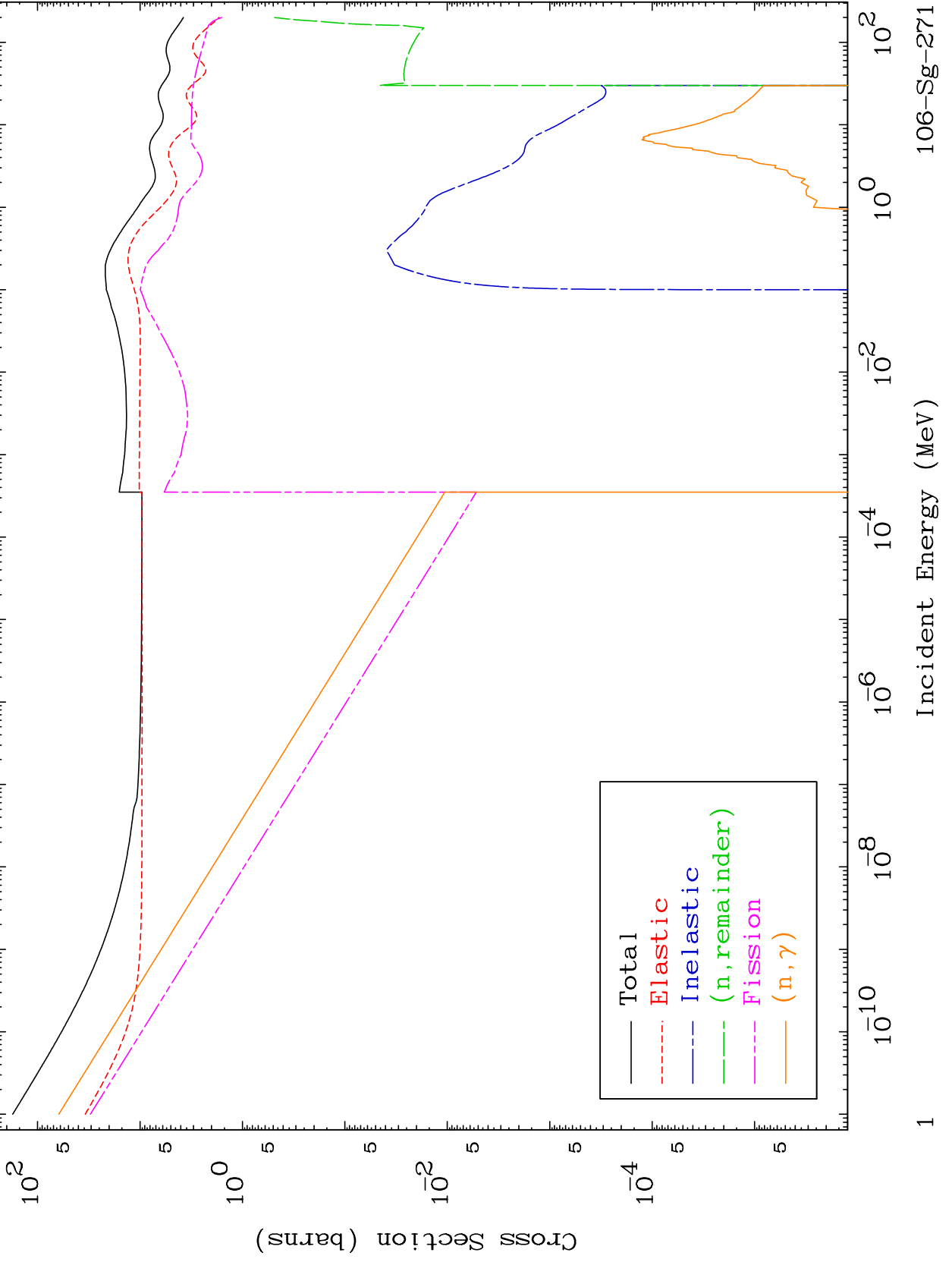
E.Mail:redcullen1@comcast.net
Web:home.comcast.net/~redcullen1

Press Mouse Button to Start

MAT 671

Major
293 Kelvin Cross Sections

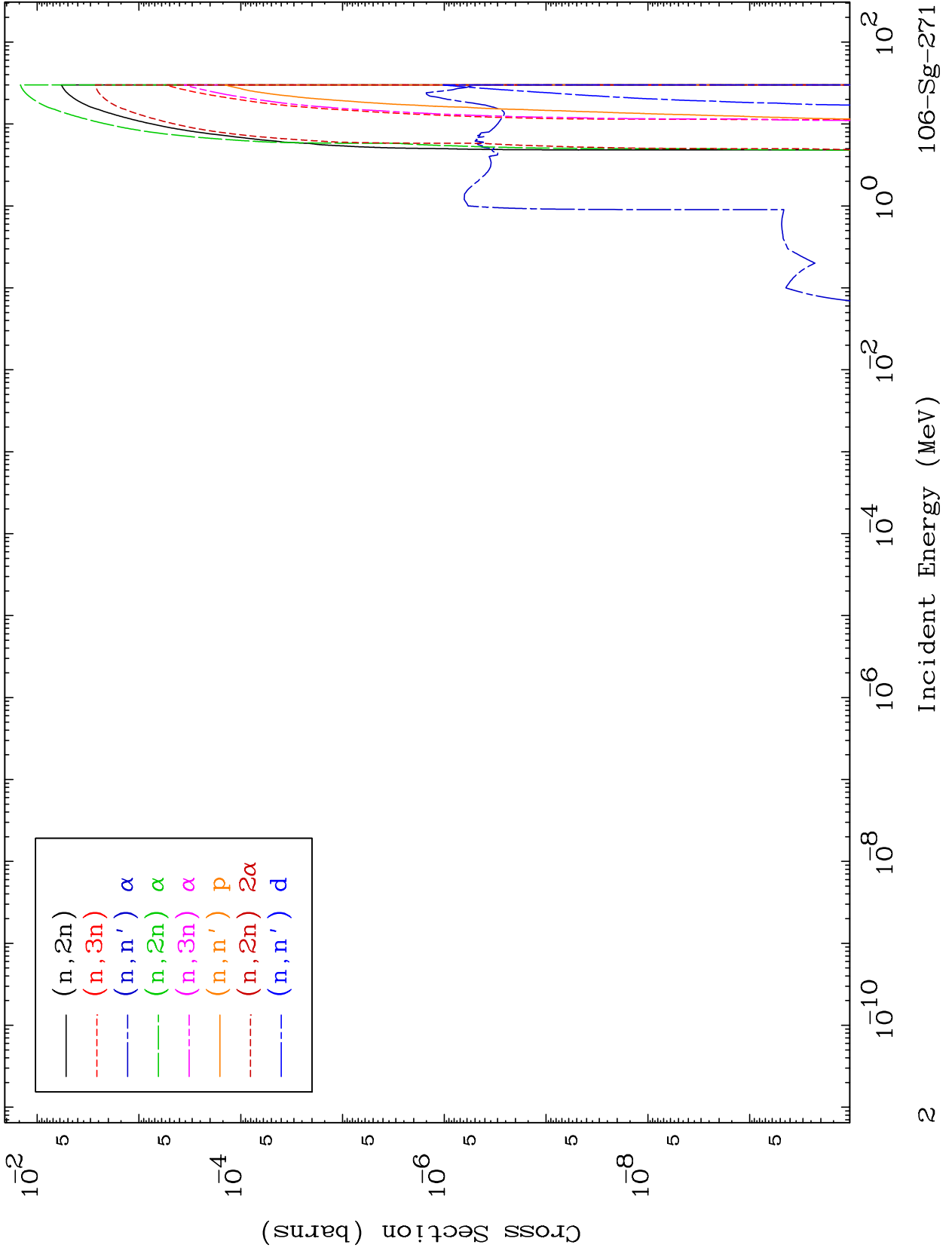
106-Sg-271

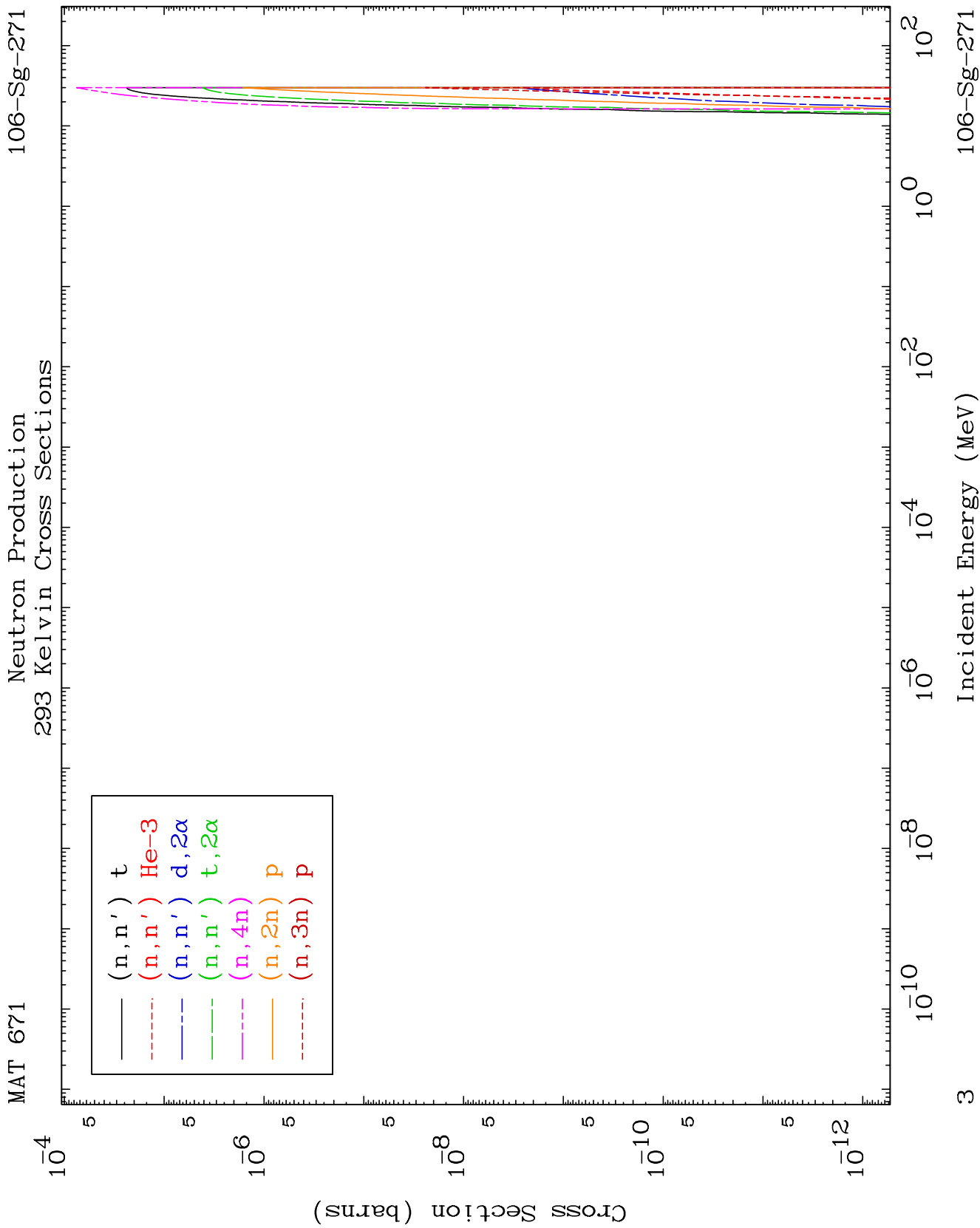


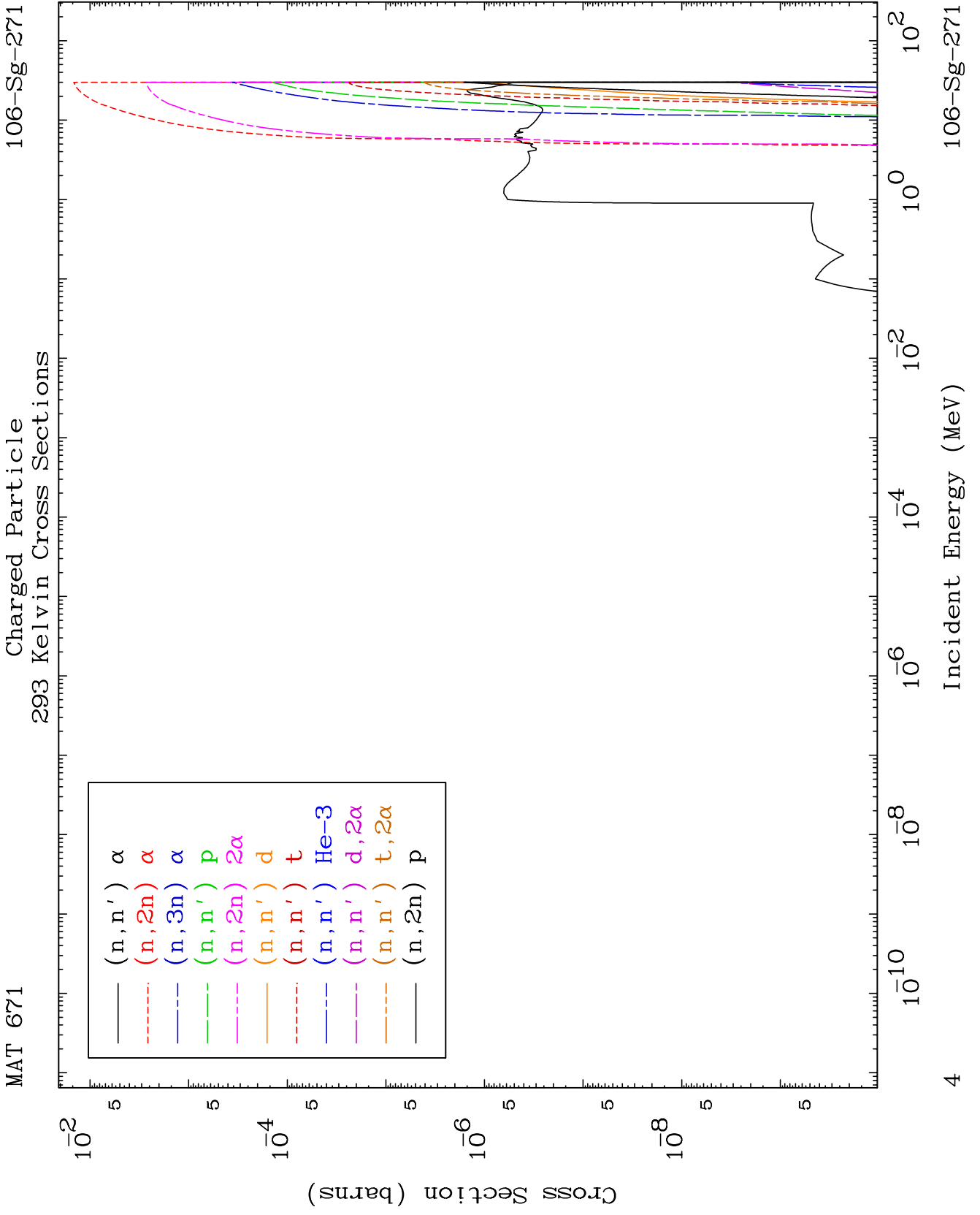
MAT 671

Neutron Production
293 Kelvin Cross Sections

106-Sg-271



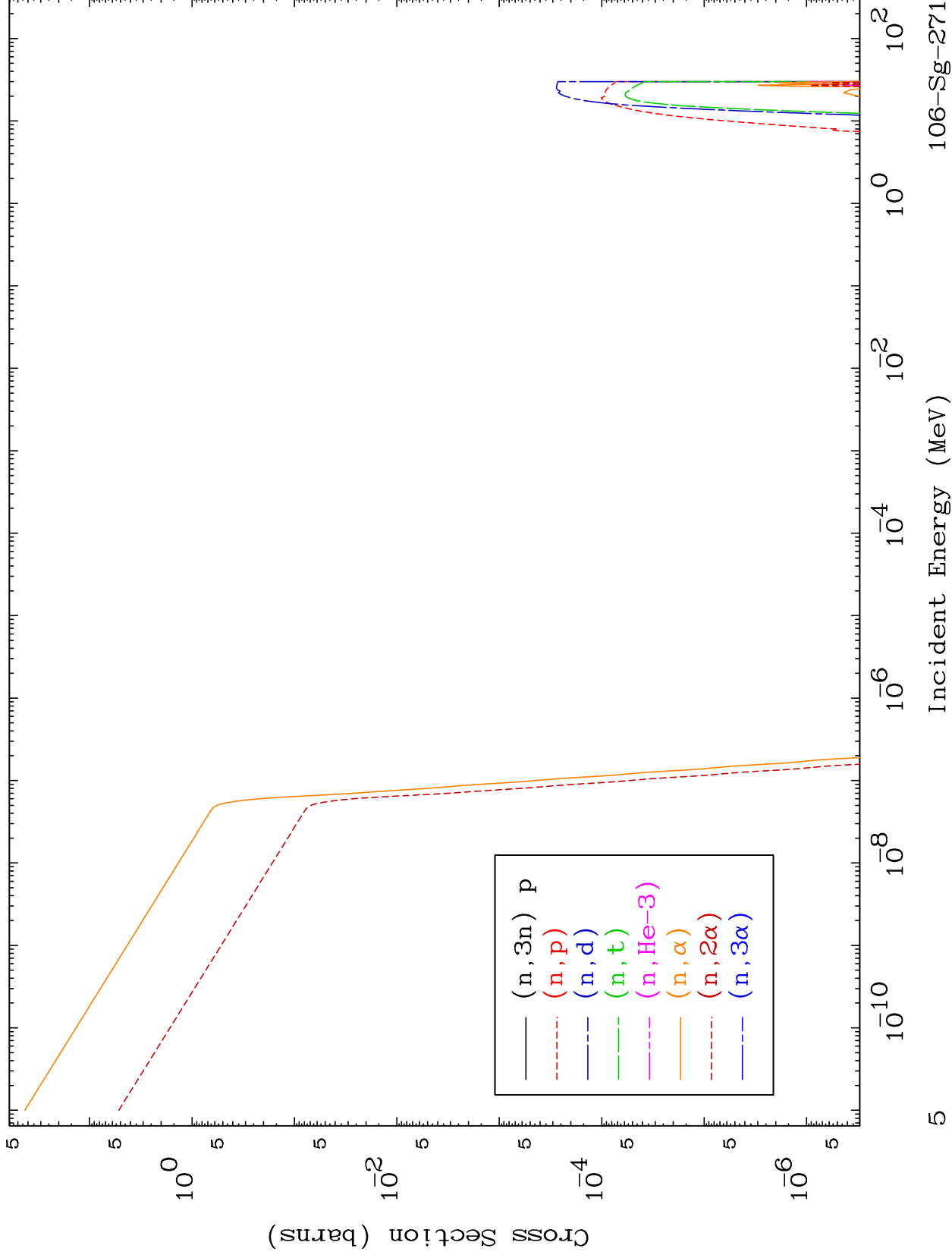


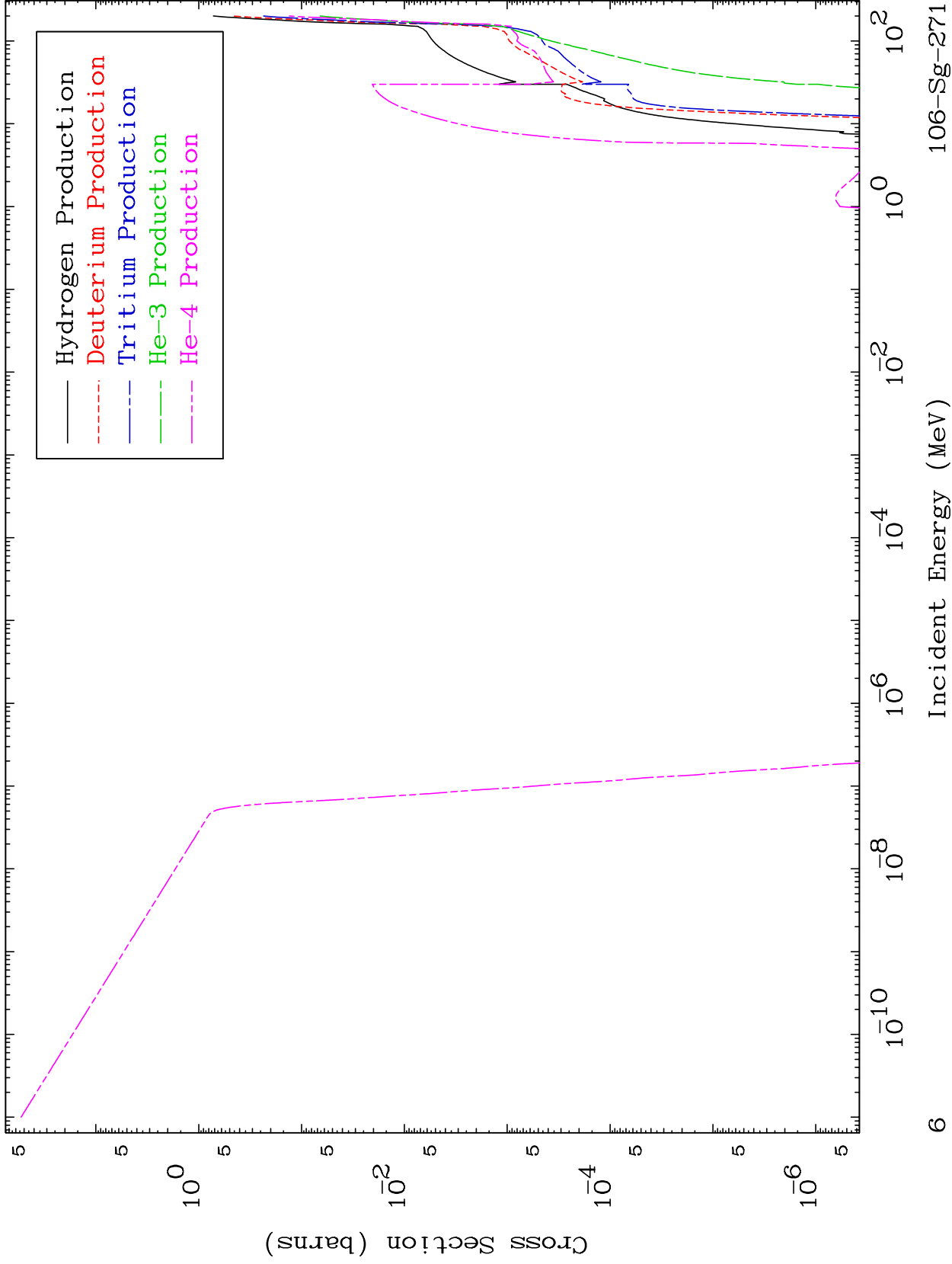


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Charged Particle
293 Kelvin Cross Sections

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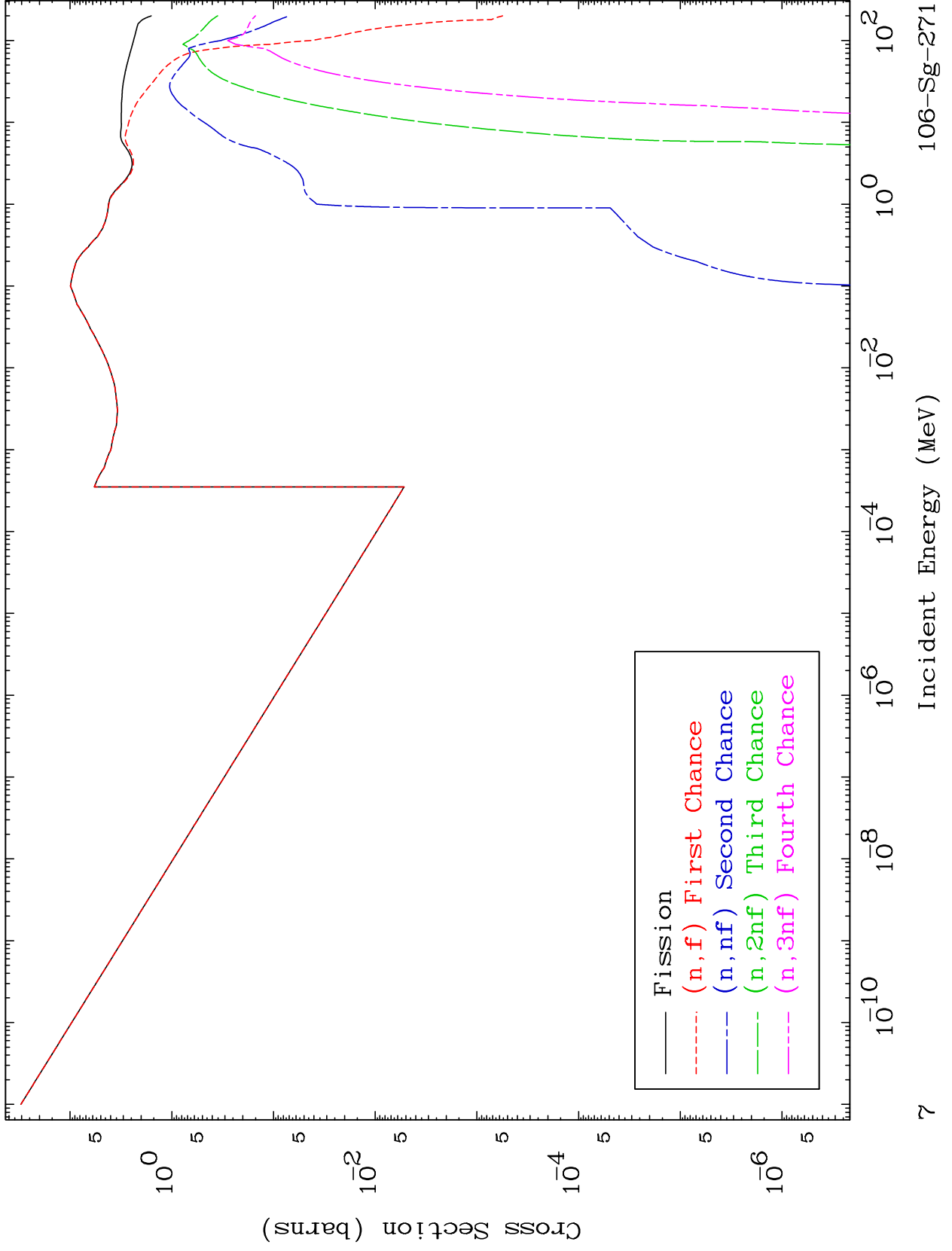


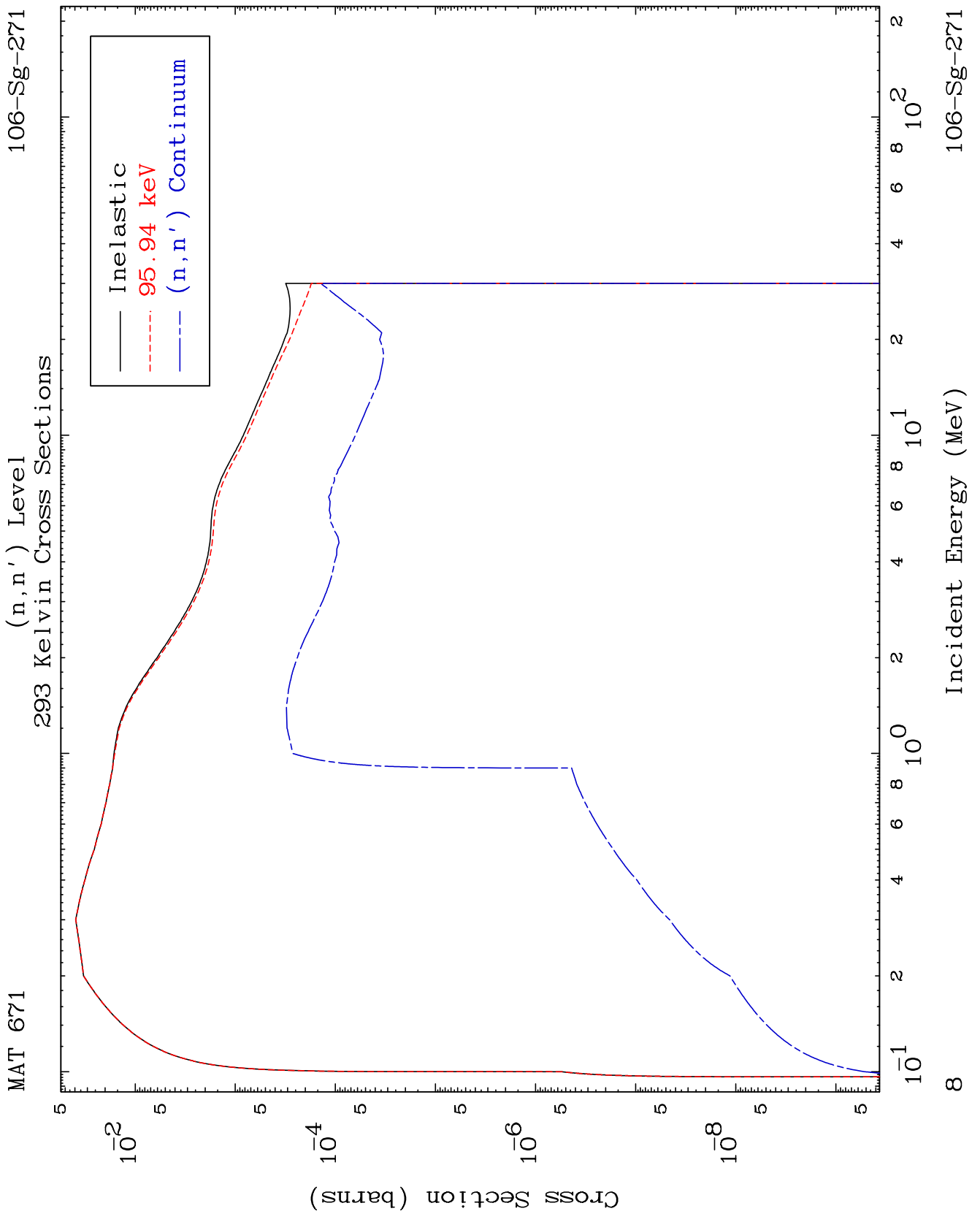


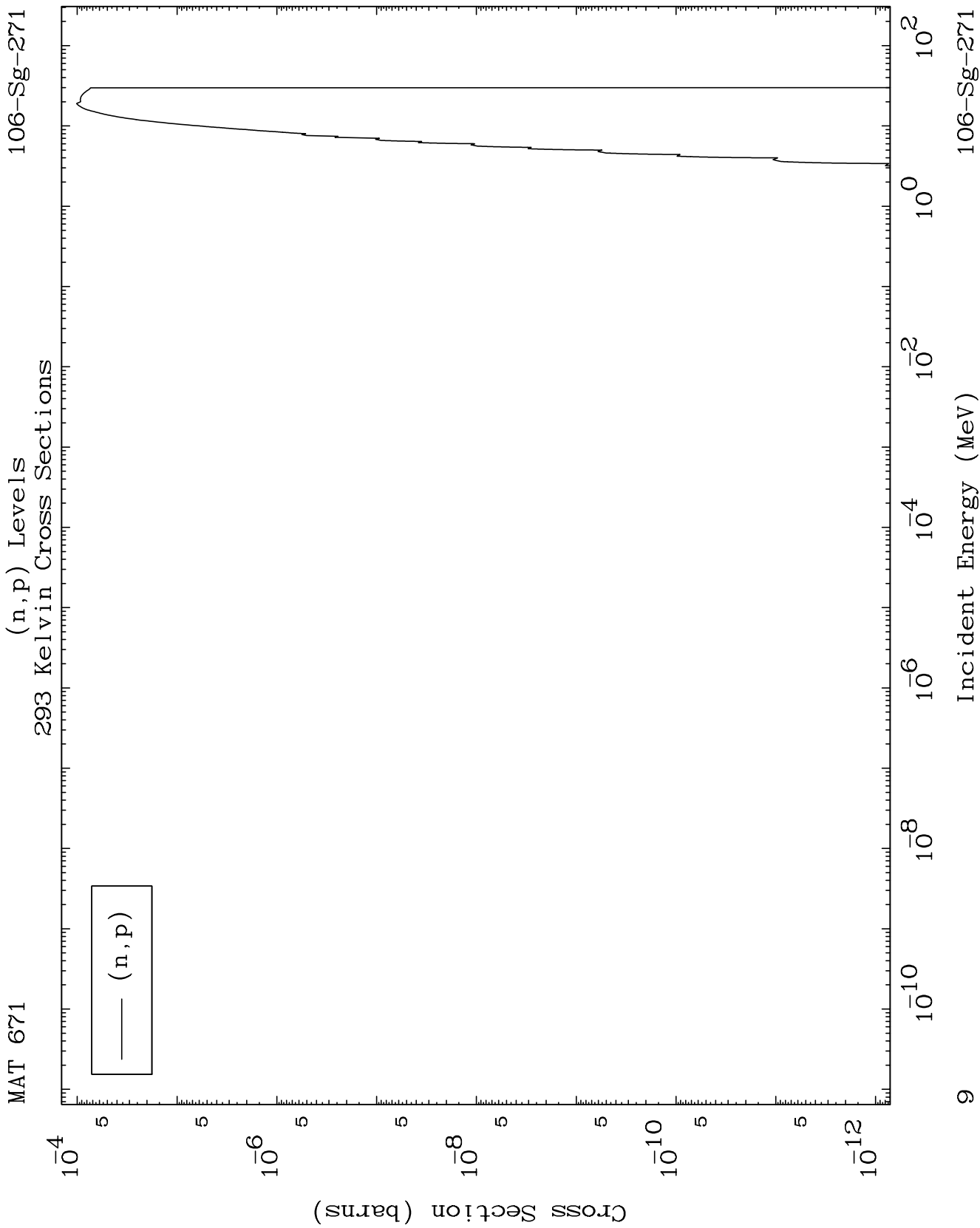
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Fission
293 Kelvin Cross Sections

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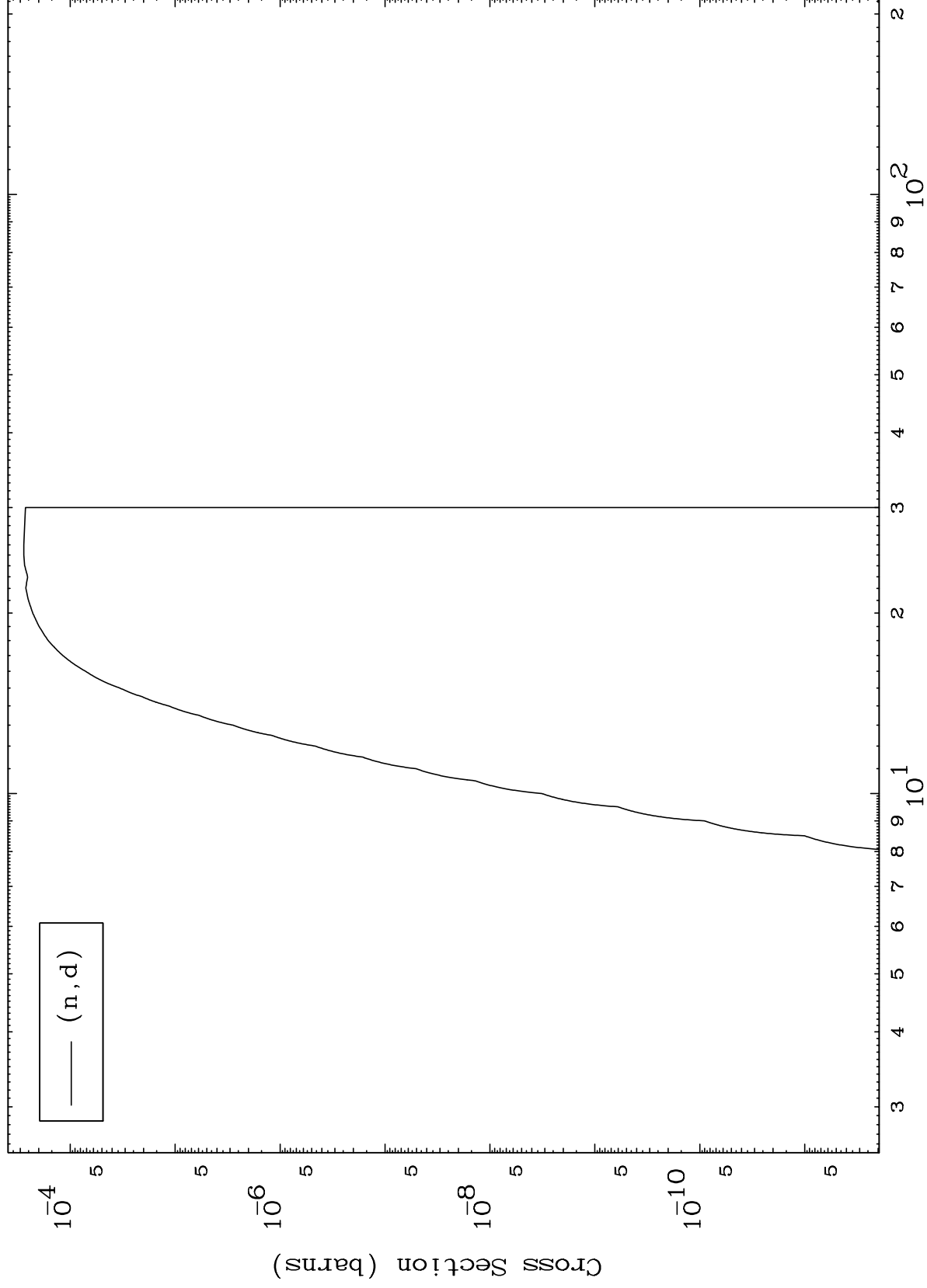




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(n,d) Levels
293 Kelvin Cross Sections

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10

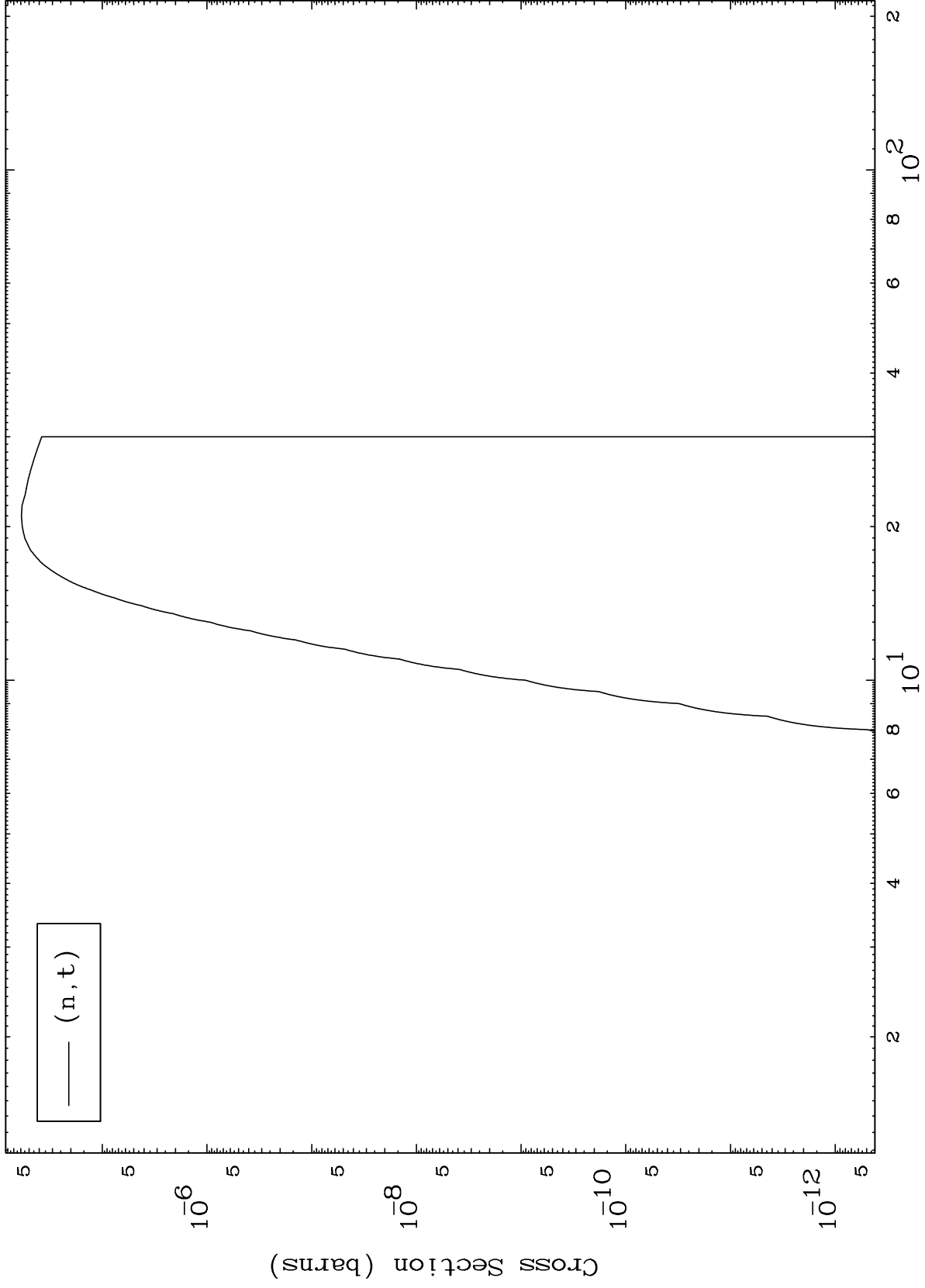
Incident Energy (MeV)

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(n,t) Levels
293 Kelvin Cross Sections

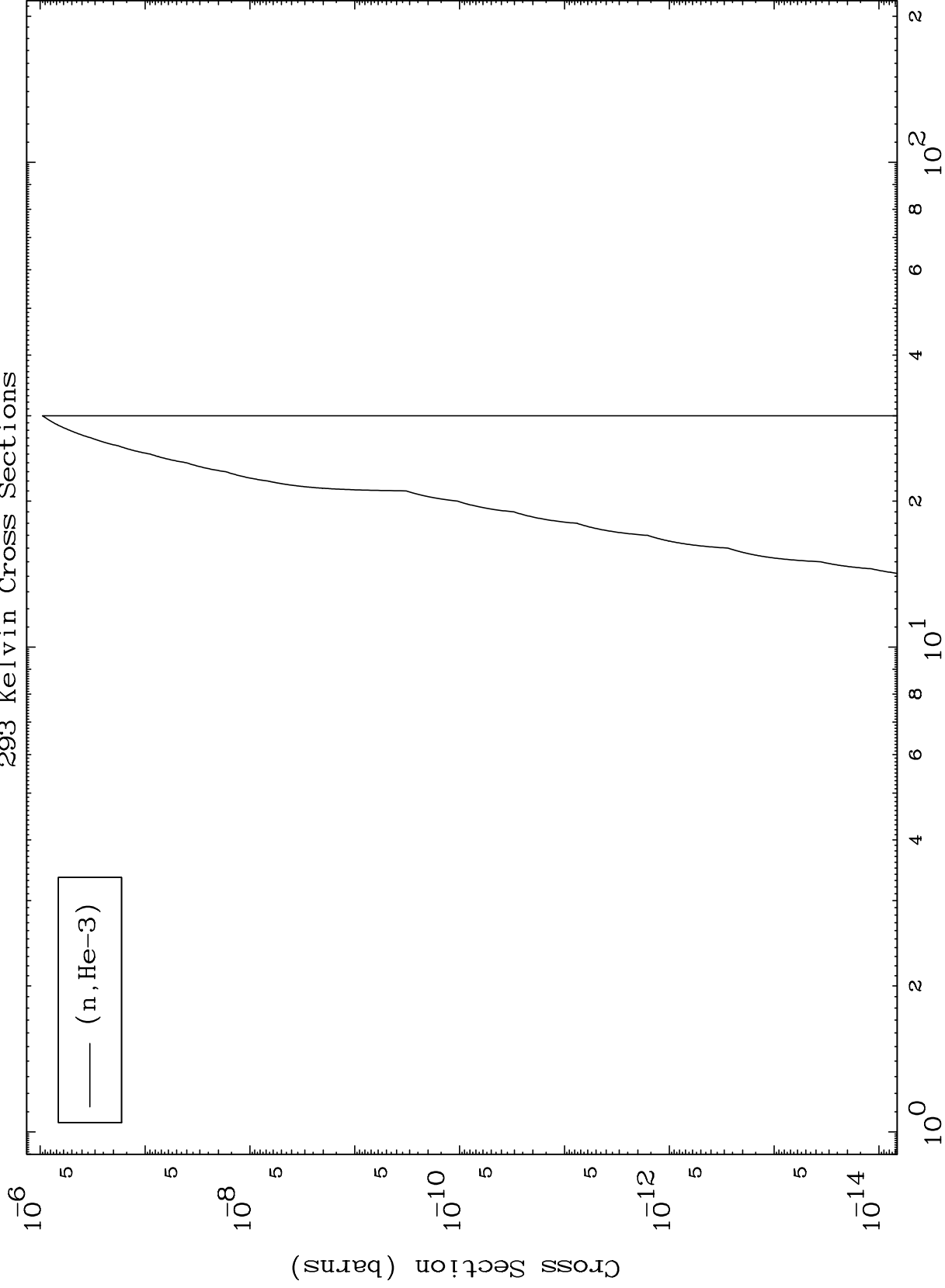
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(n,He3) Levels
293 Kelvin Cross Sections

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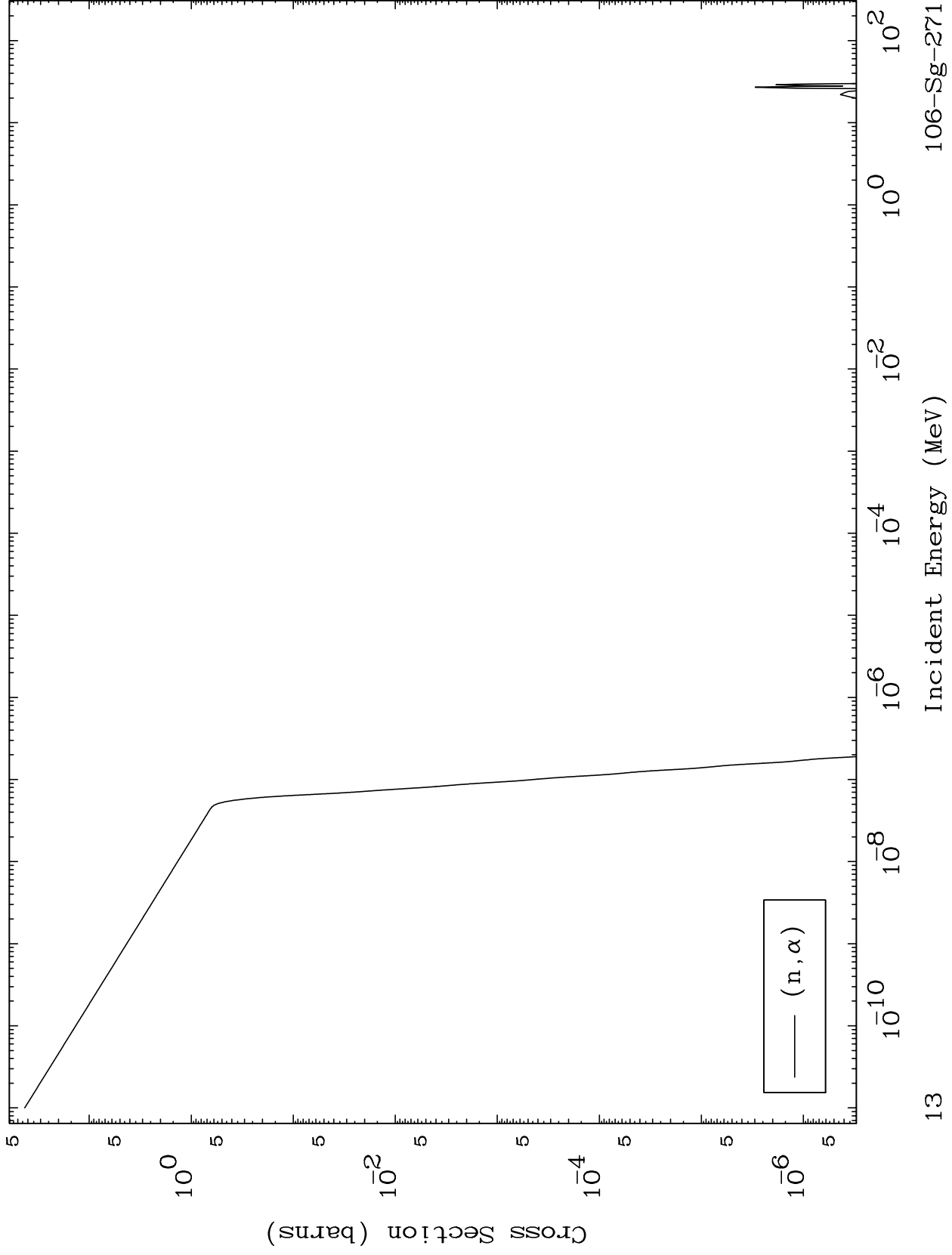
Incident Energy (MeV)

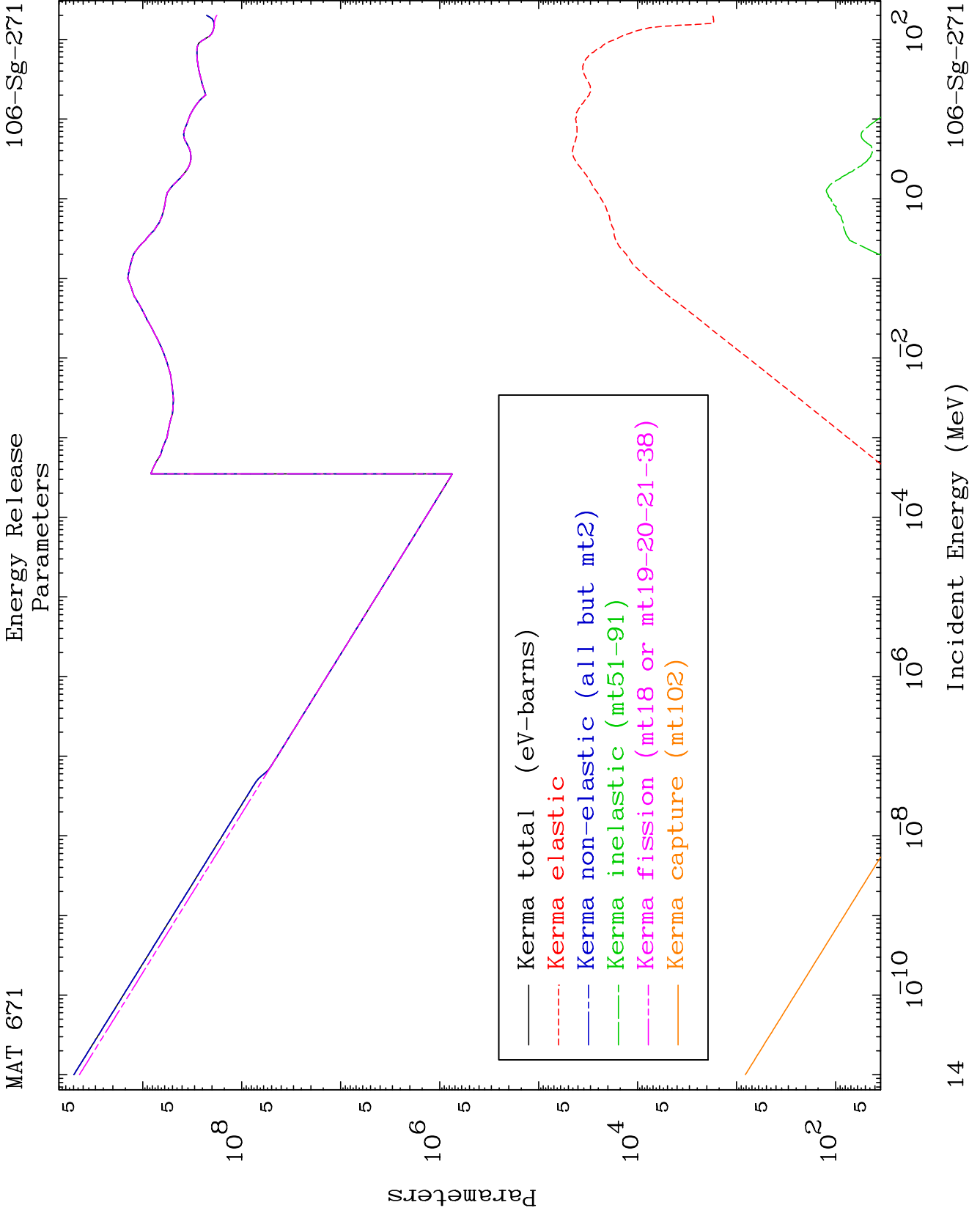
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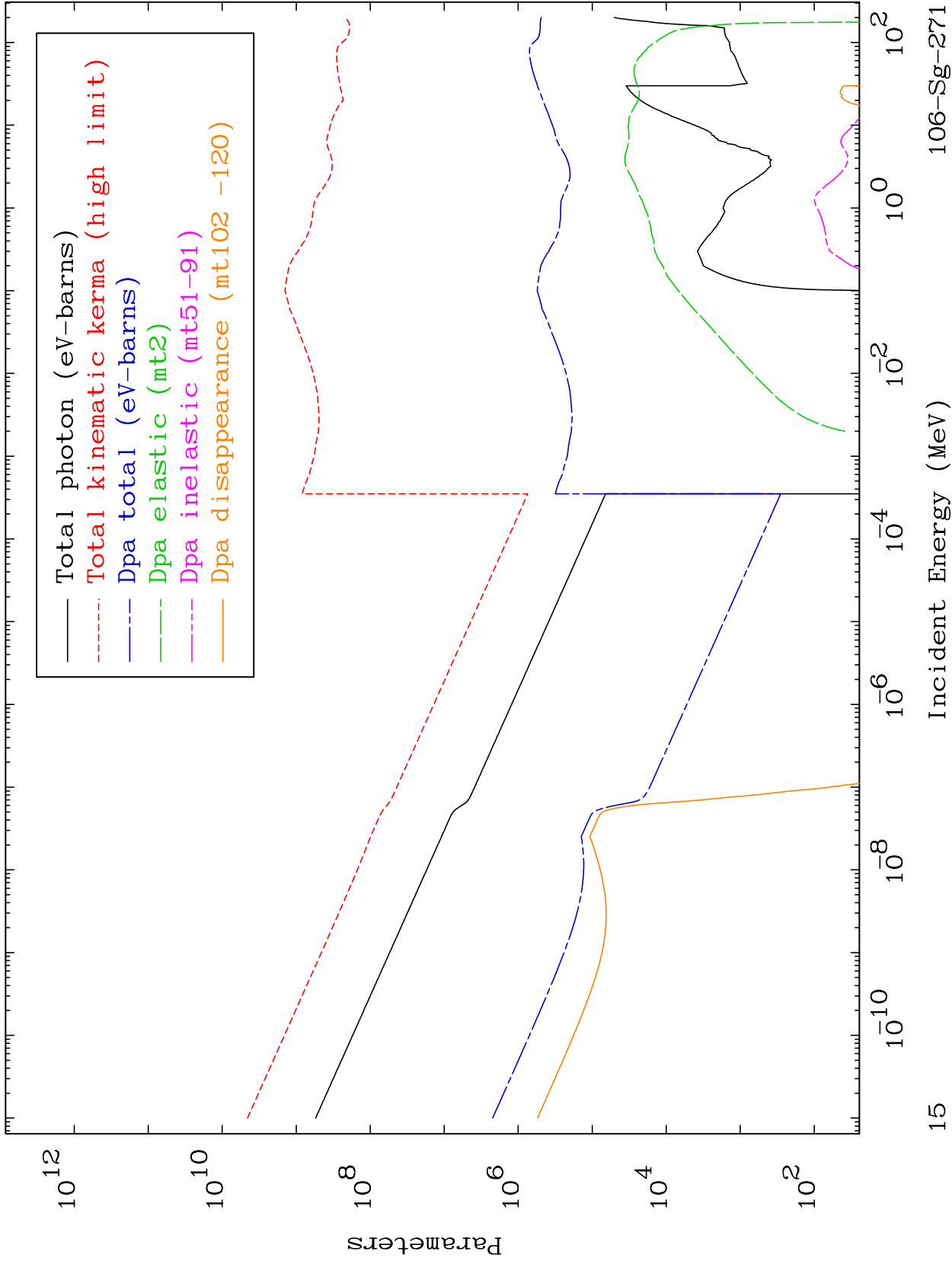
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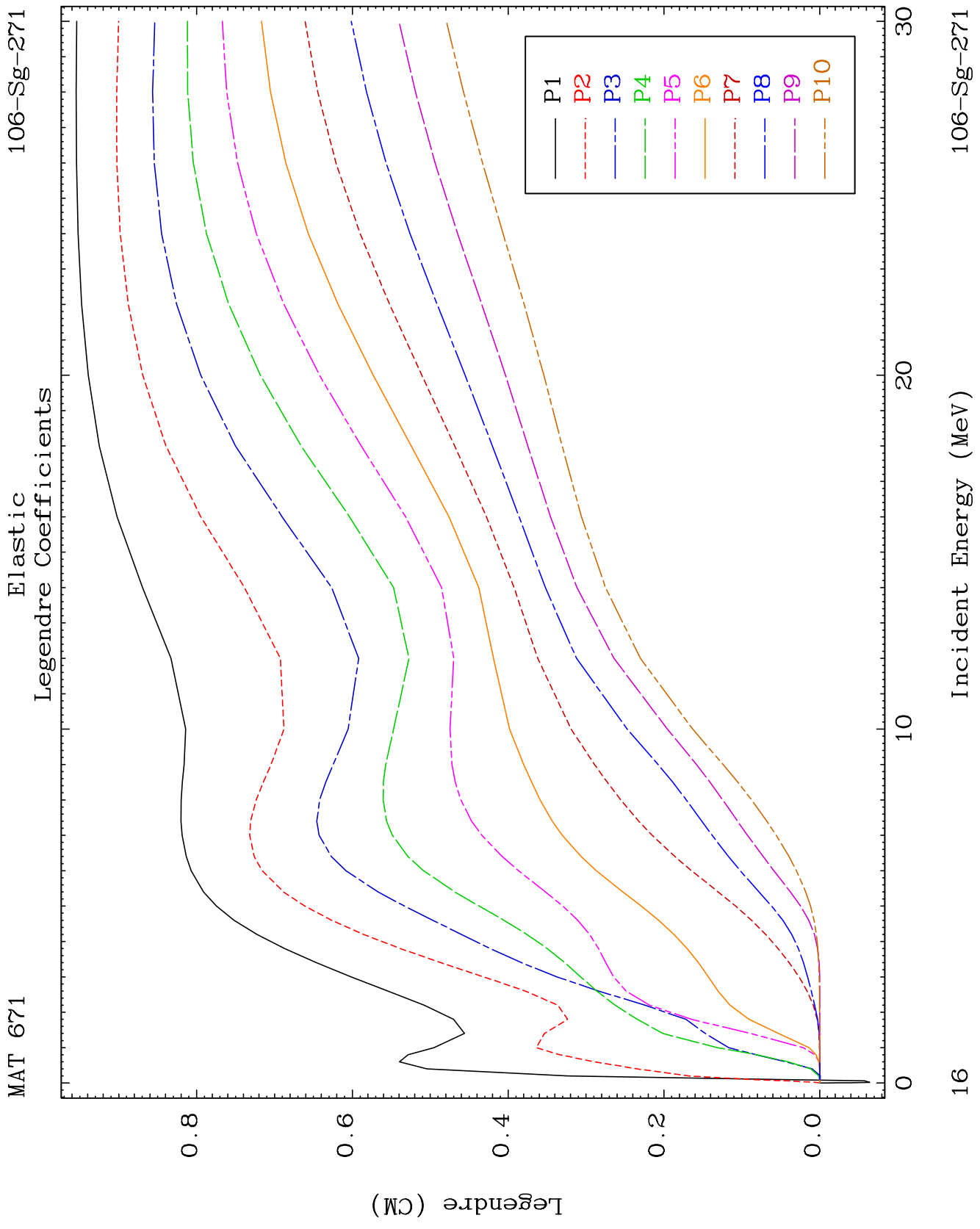
(n,α) Levels
293 Kelvin Cross Sections

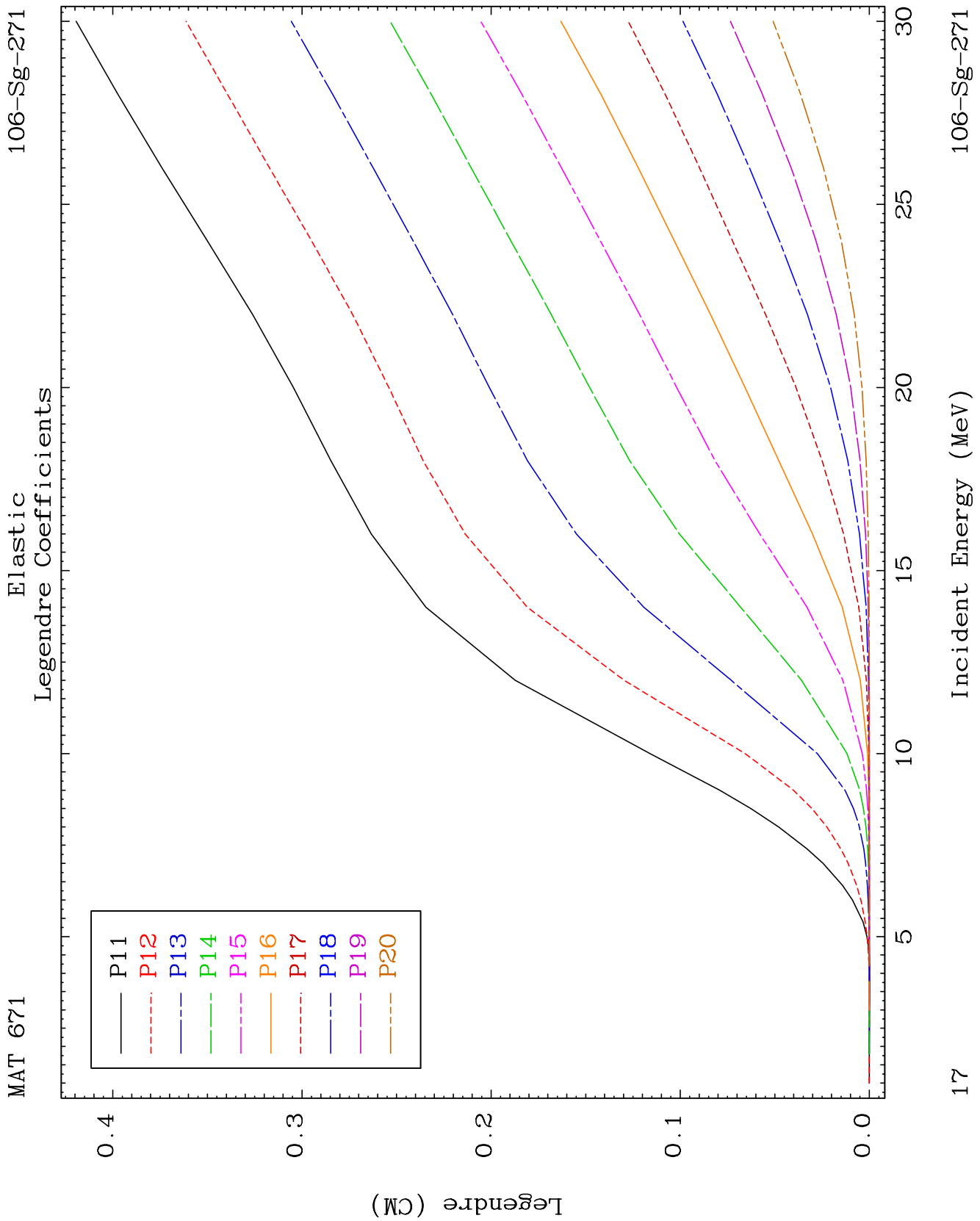
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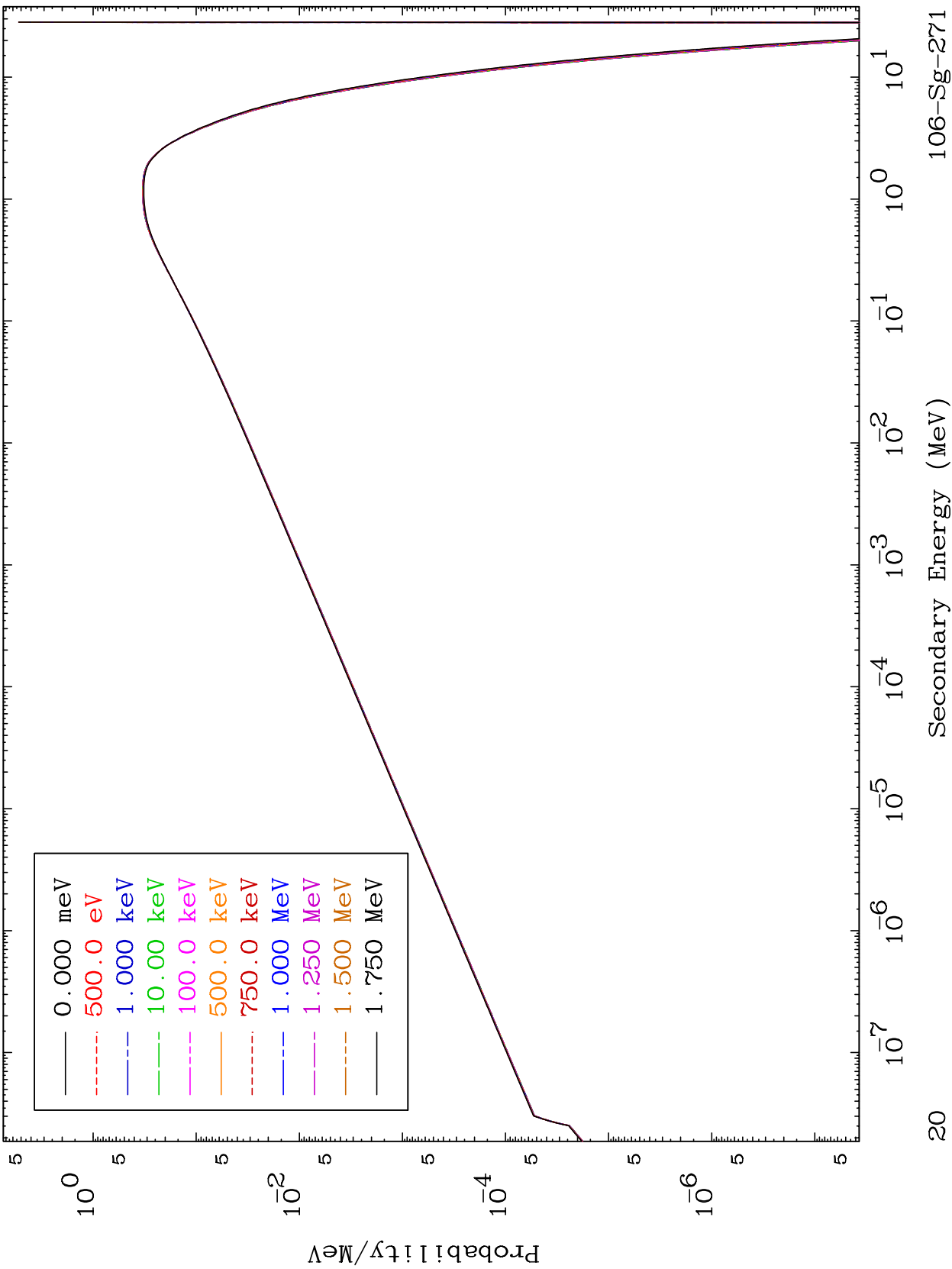




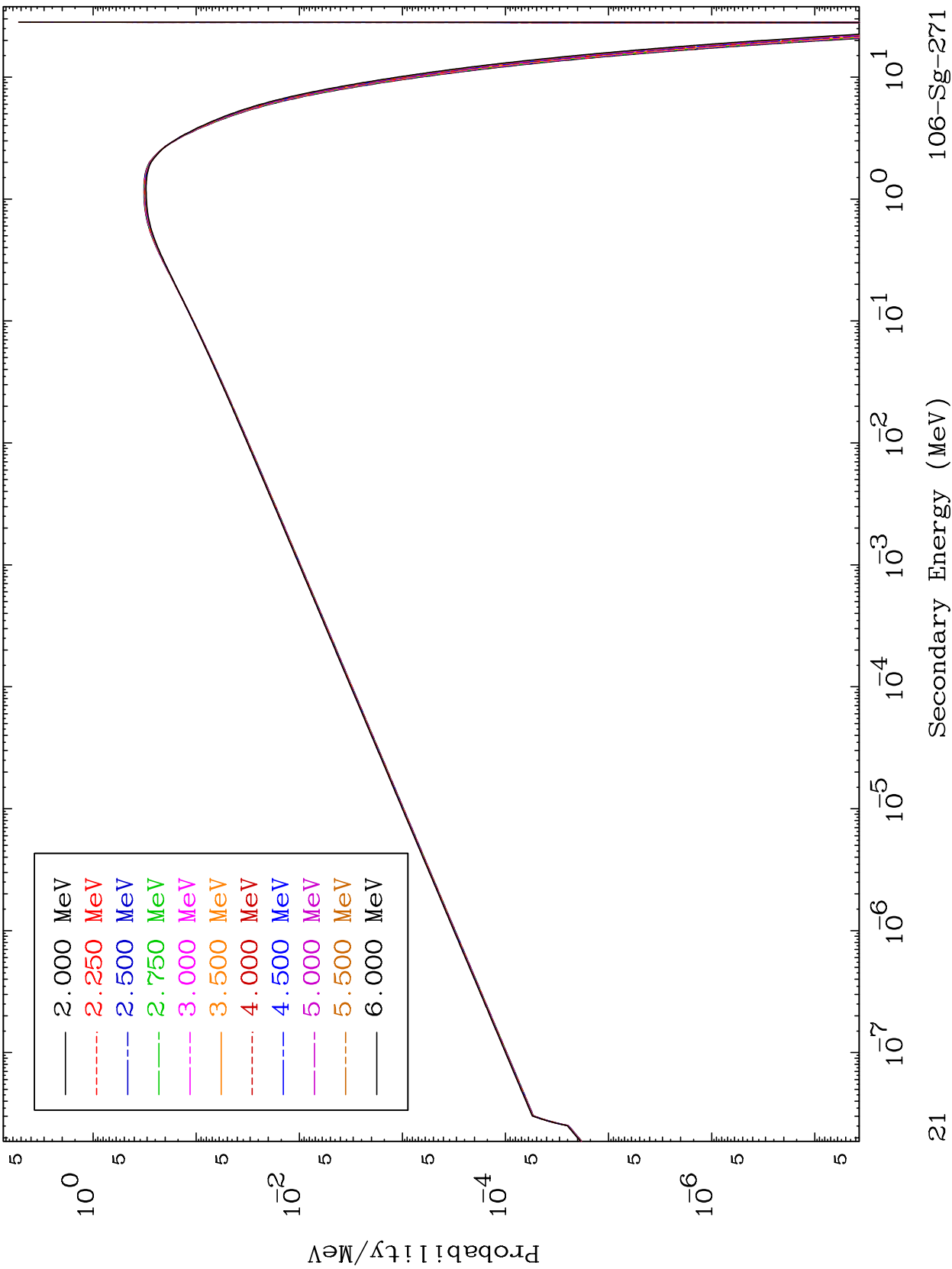








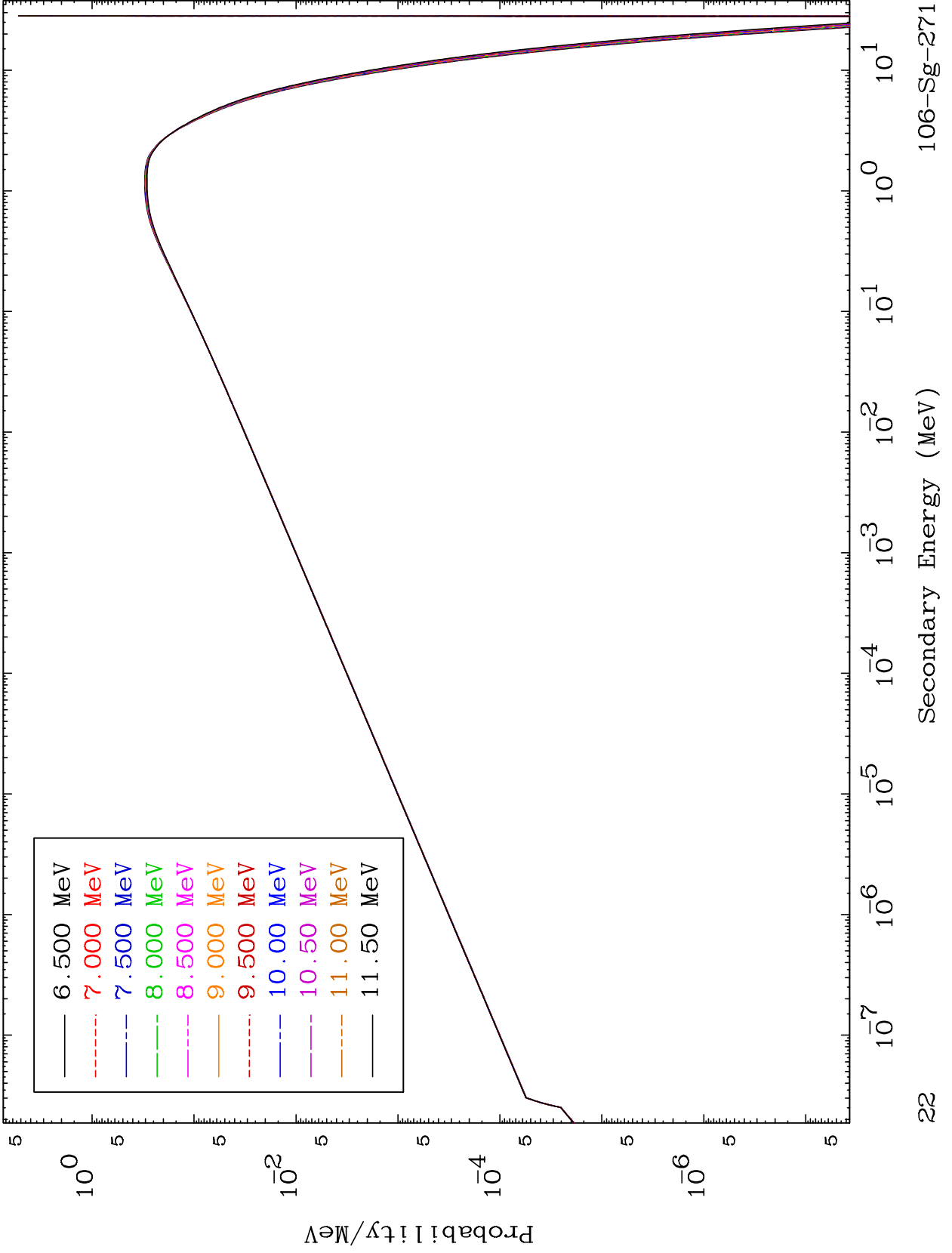
Fission
Energy Distribution



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Fission Energy Distribution

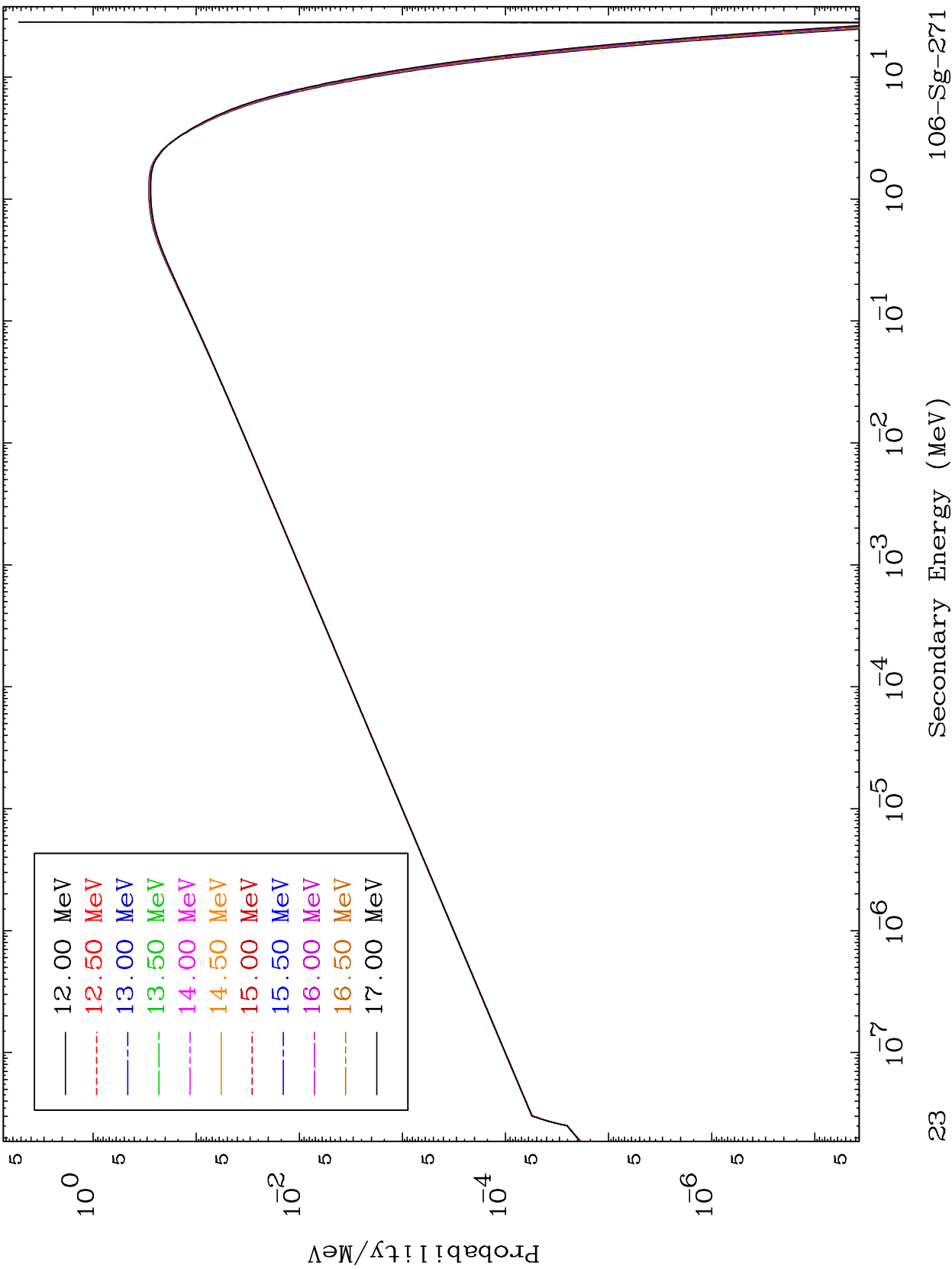
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Fission
Energy Distribution

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Secondary Energy (MeV)

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Fission Energy Distribution

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