

Program EVALPLOT
(Version 2015-2)

by

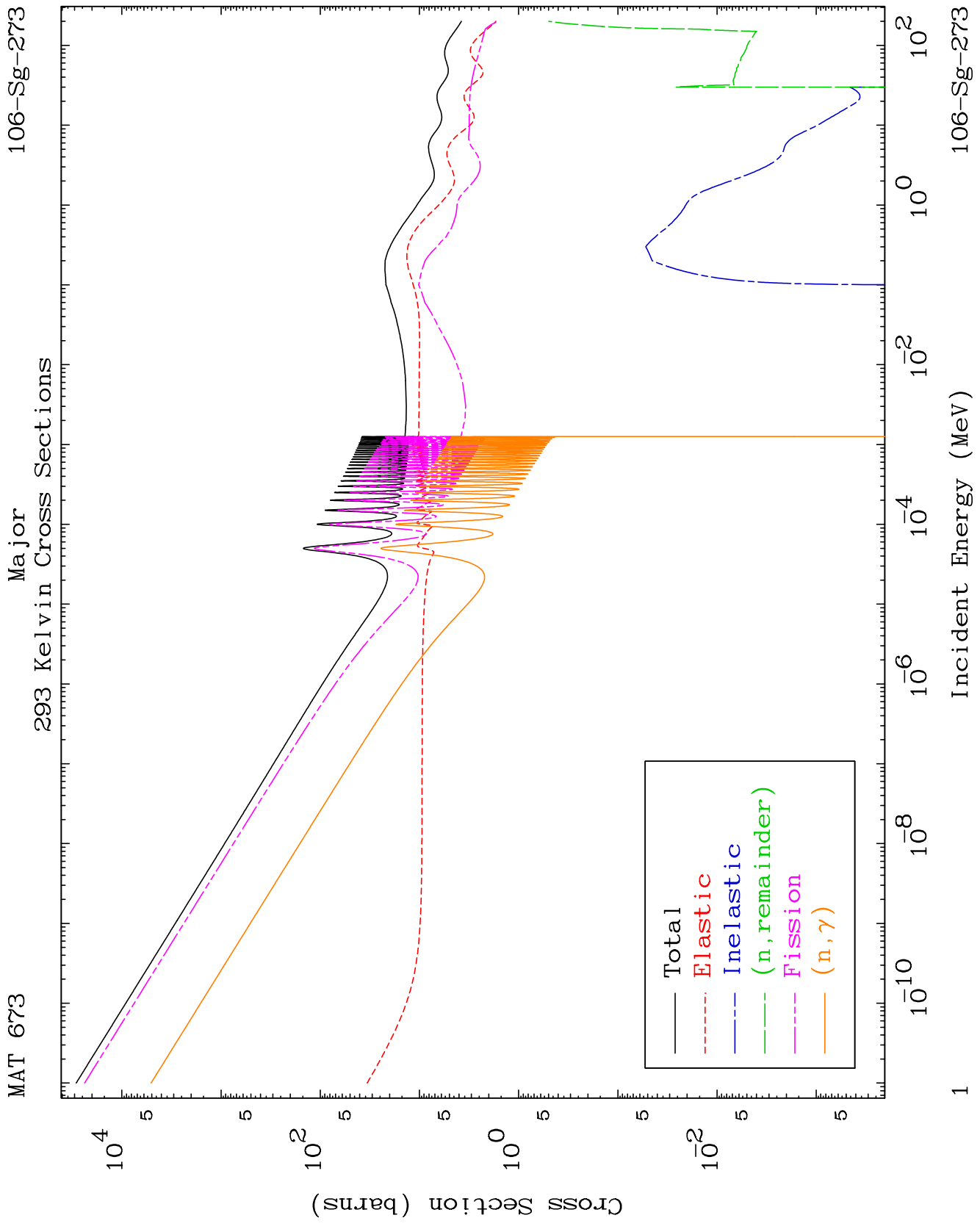
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(Present Contact Information)

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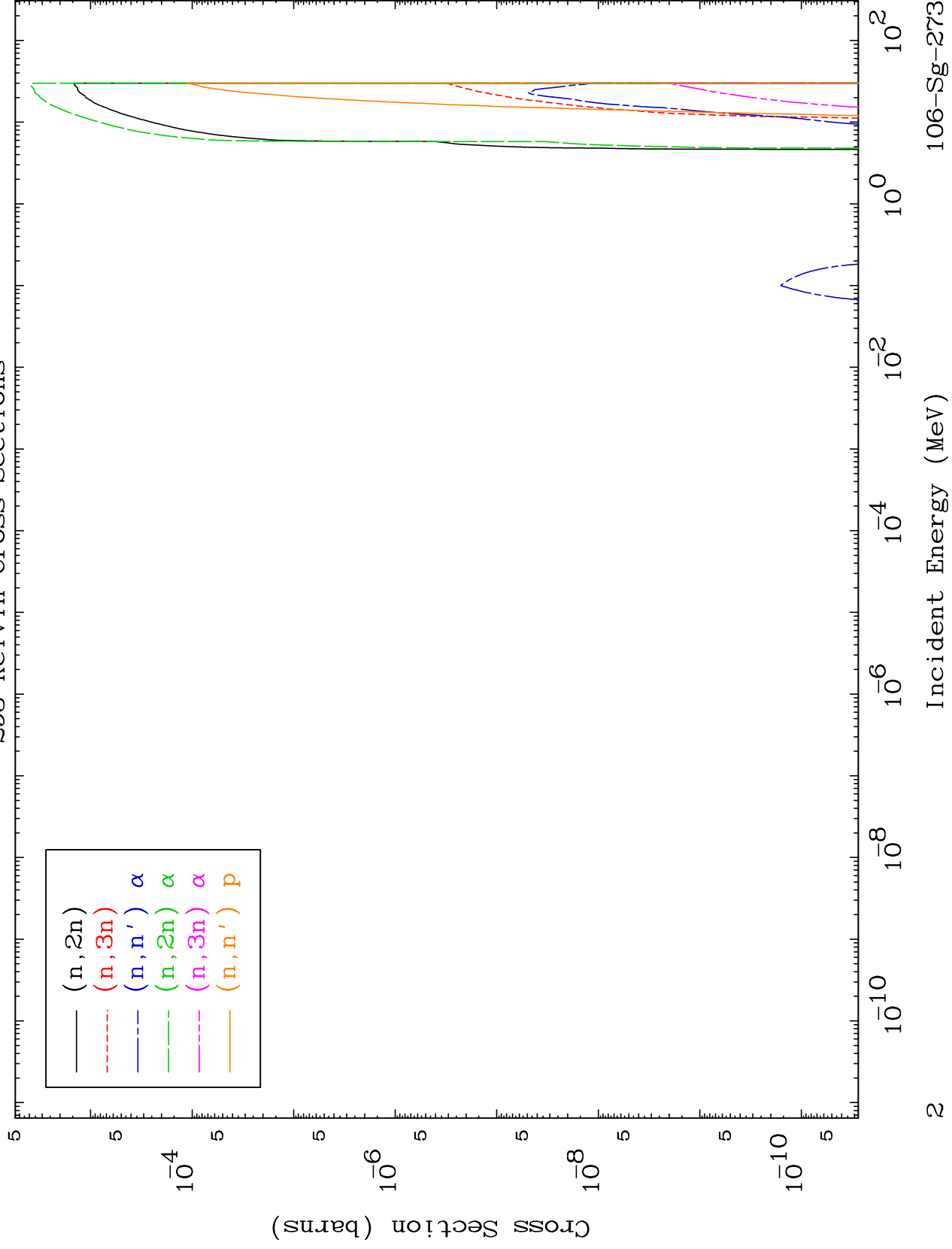
Press Mouse Button to Start



MAT 673

Neutron Production
293 Kelvin Cross Sections

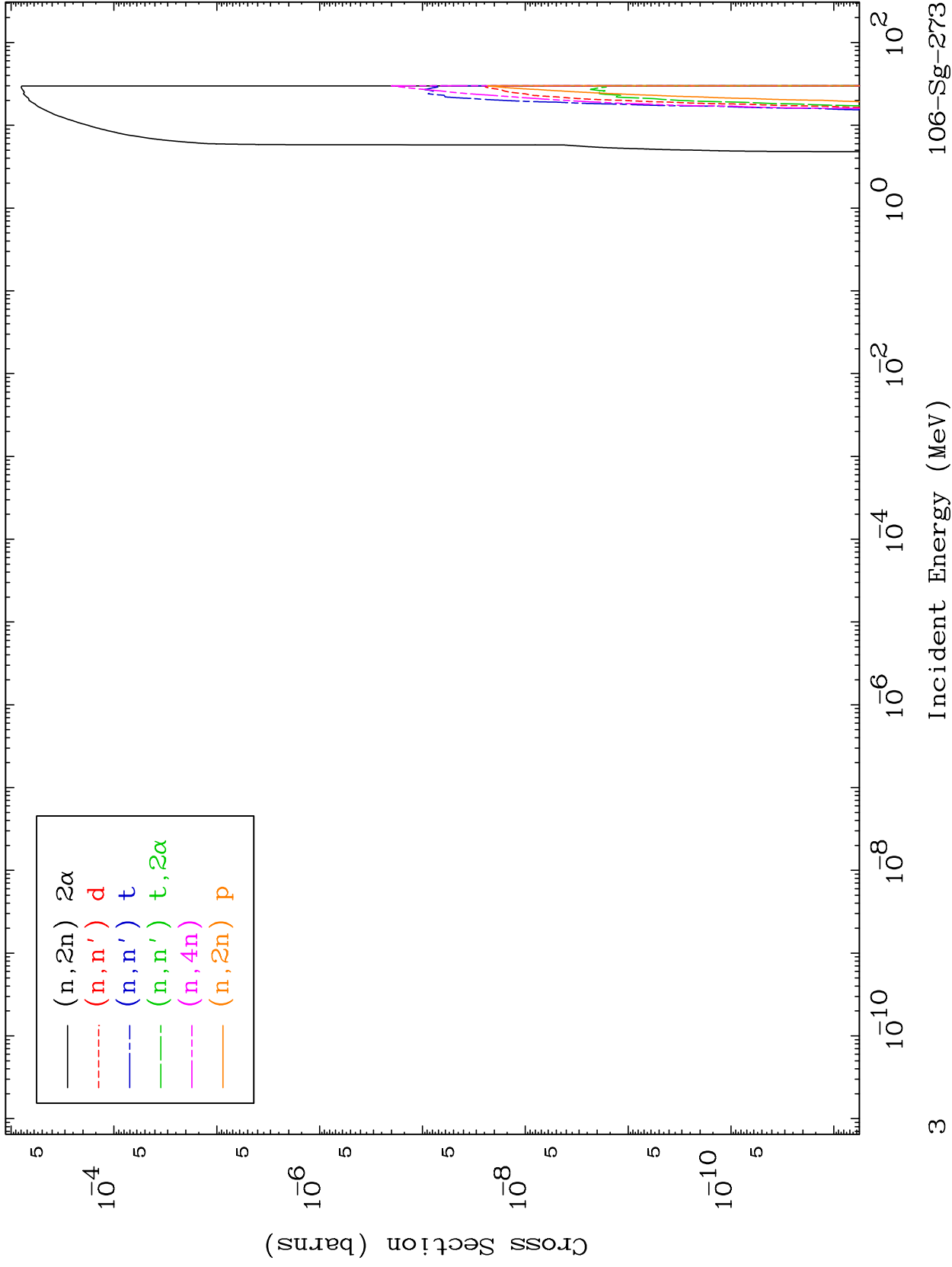
106-Sg-273



MAT 673

Neutron Production
293 Kelvin Cross Sections

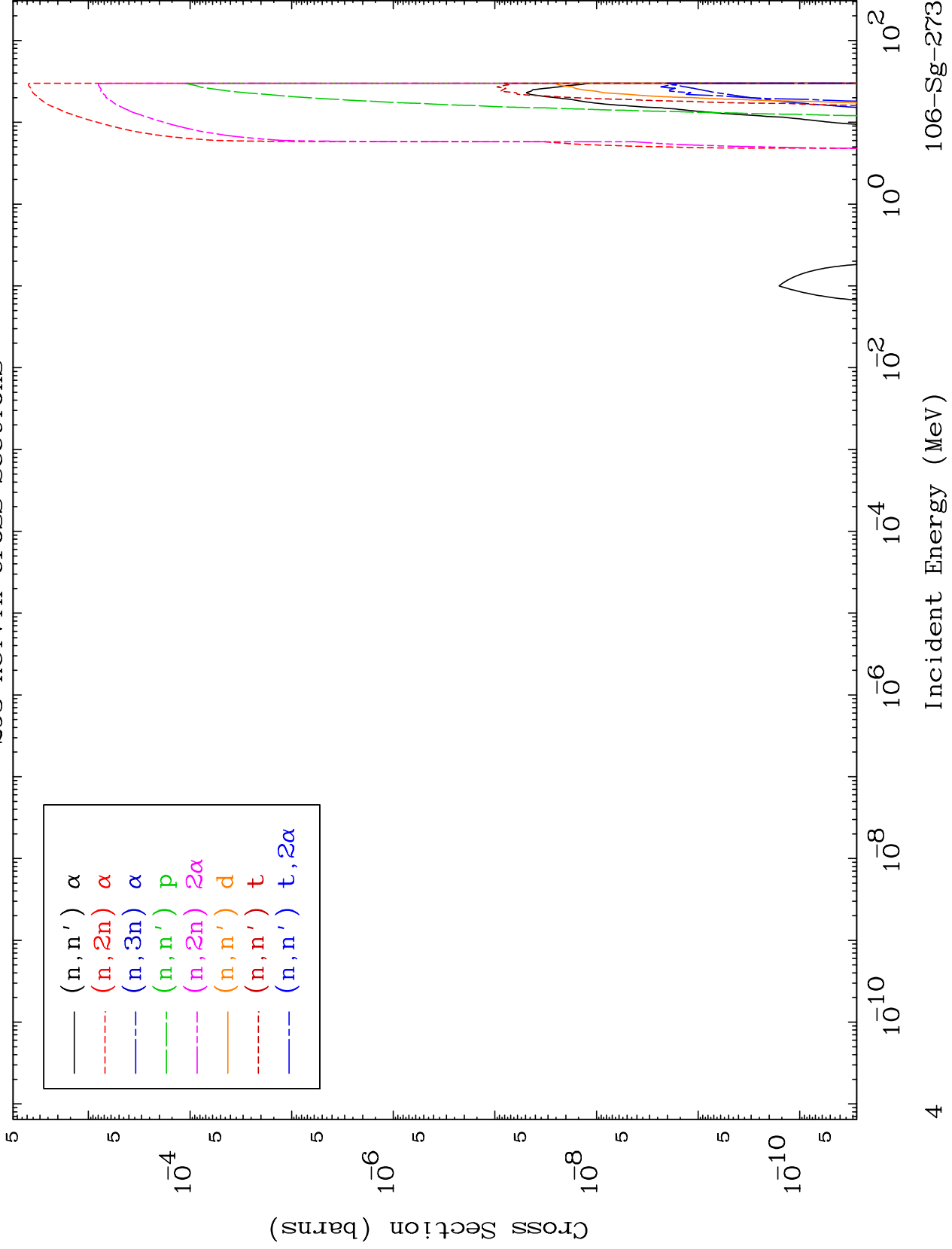
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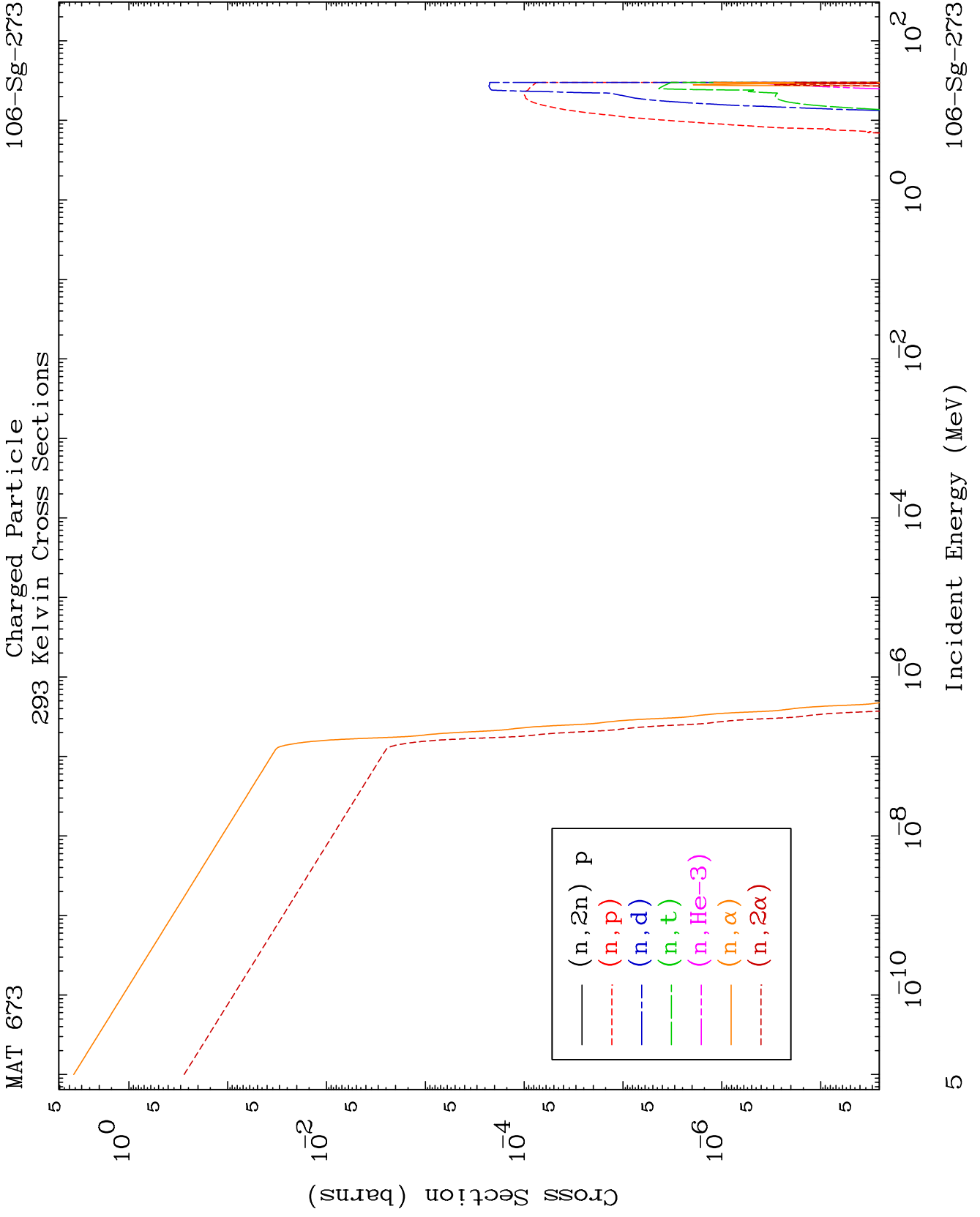


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Charged Particle
293 Kelvin Cross Sections

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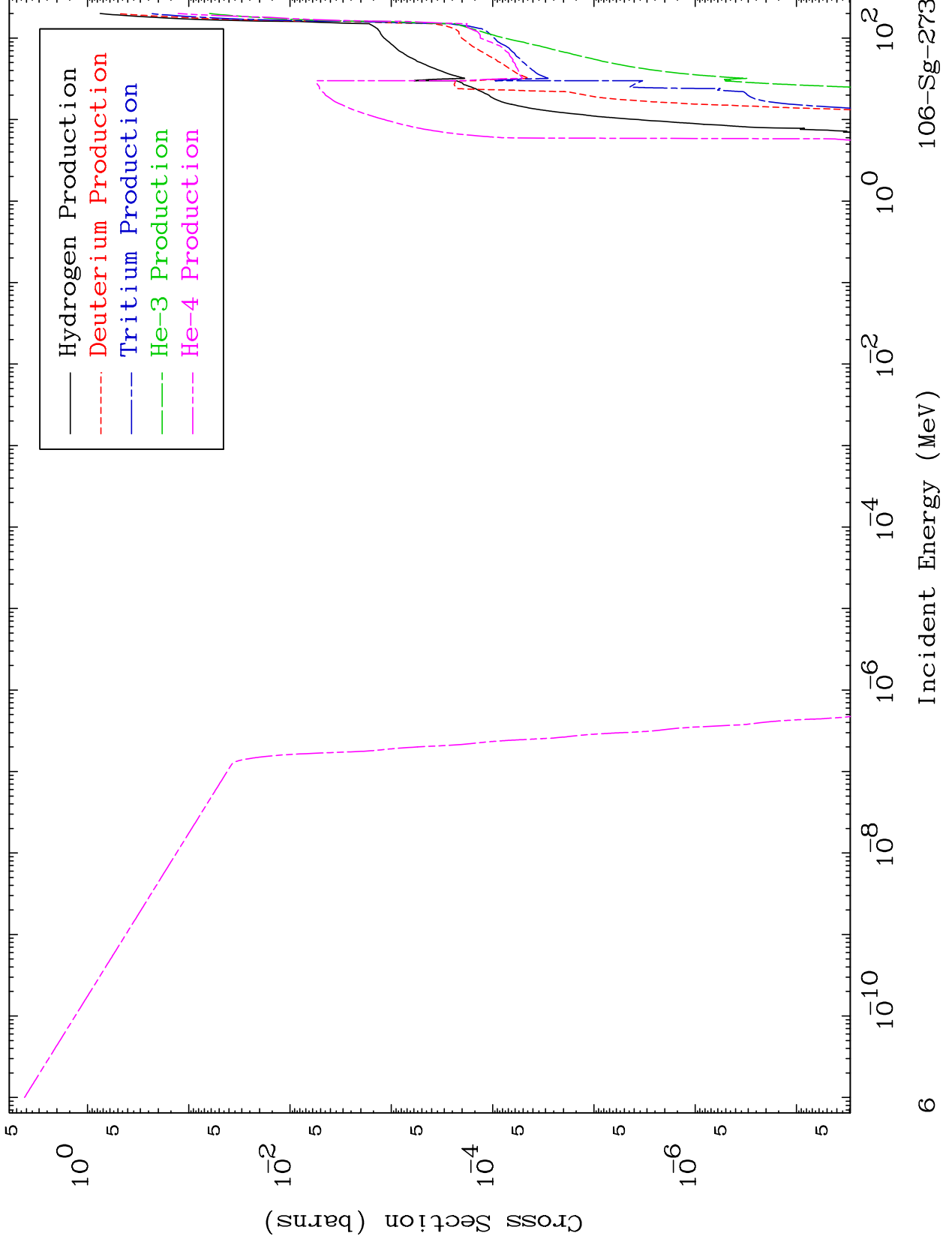




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Particle Production
293 Kelvin Cross Sections

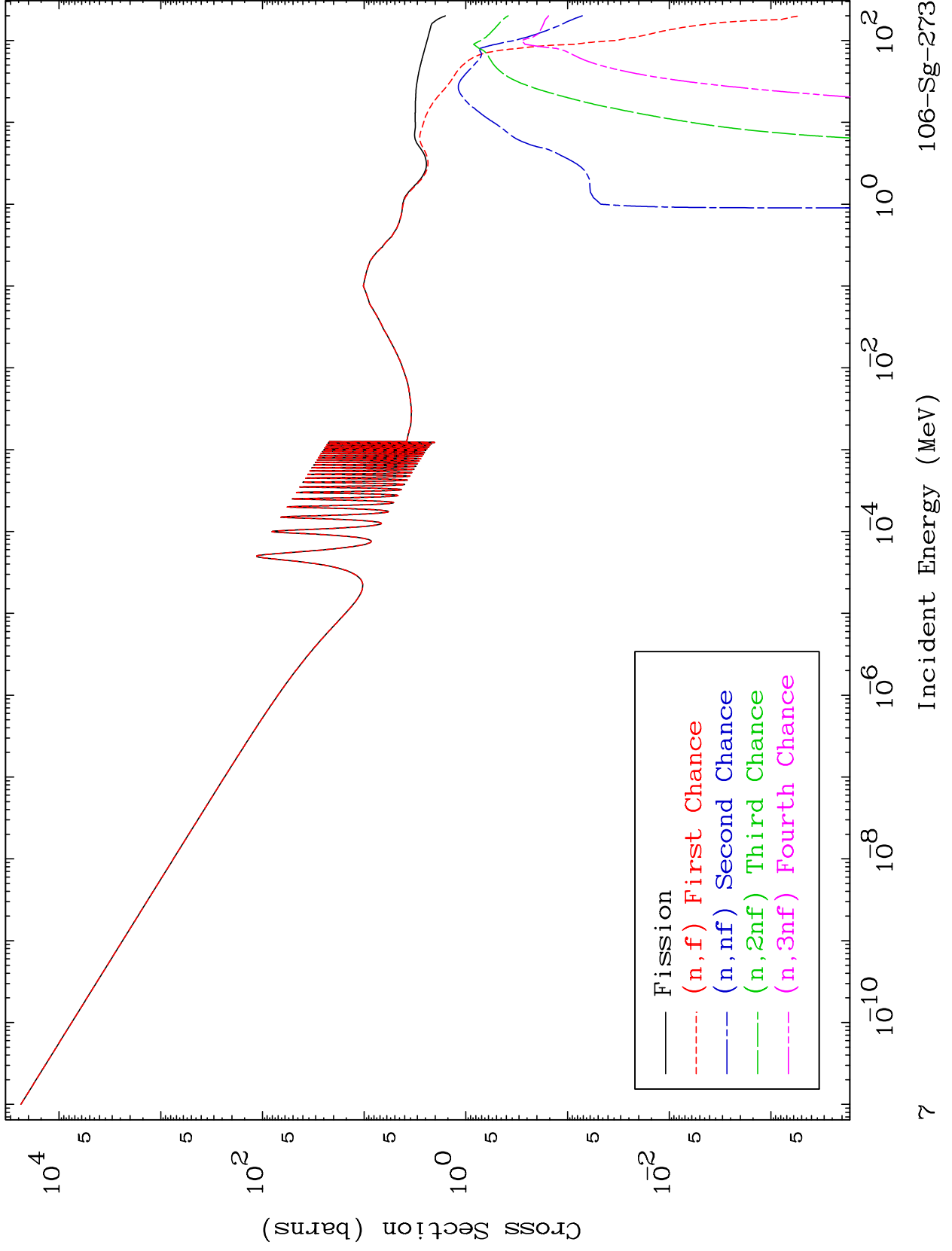
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Fission
293 Kelvin Cross Sections

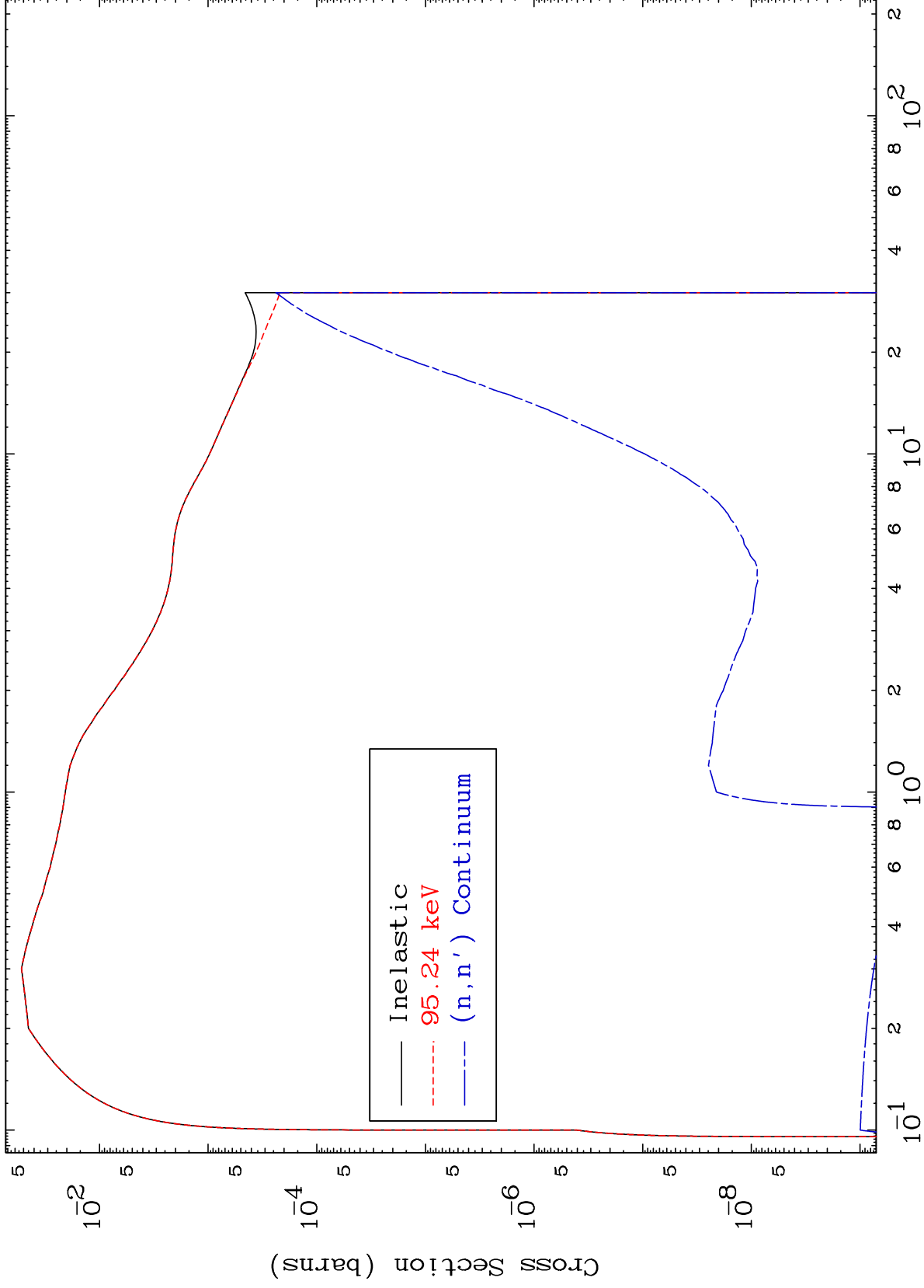
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106-Sg-273

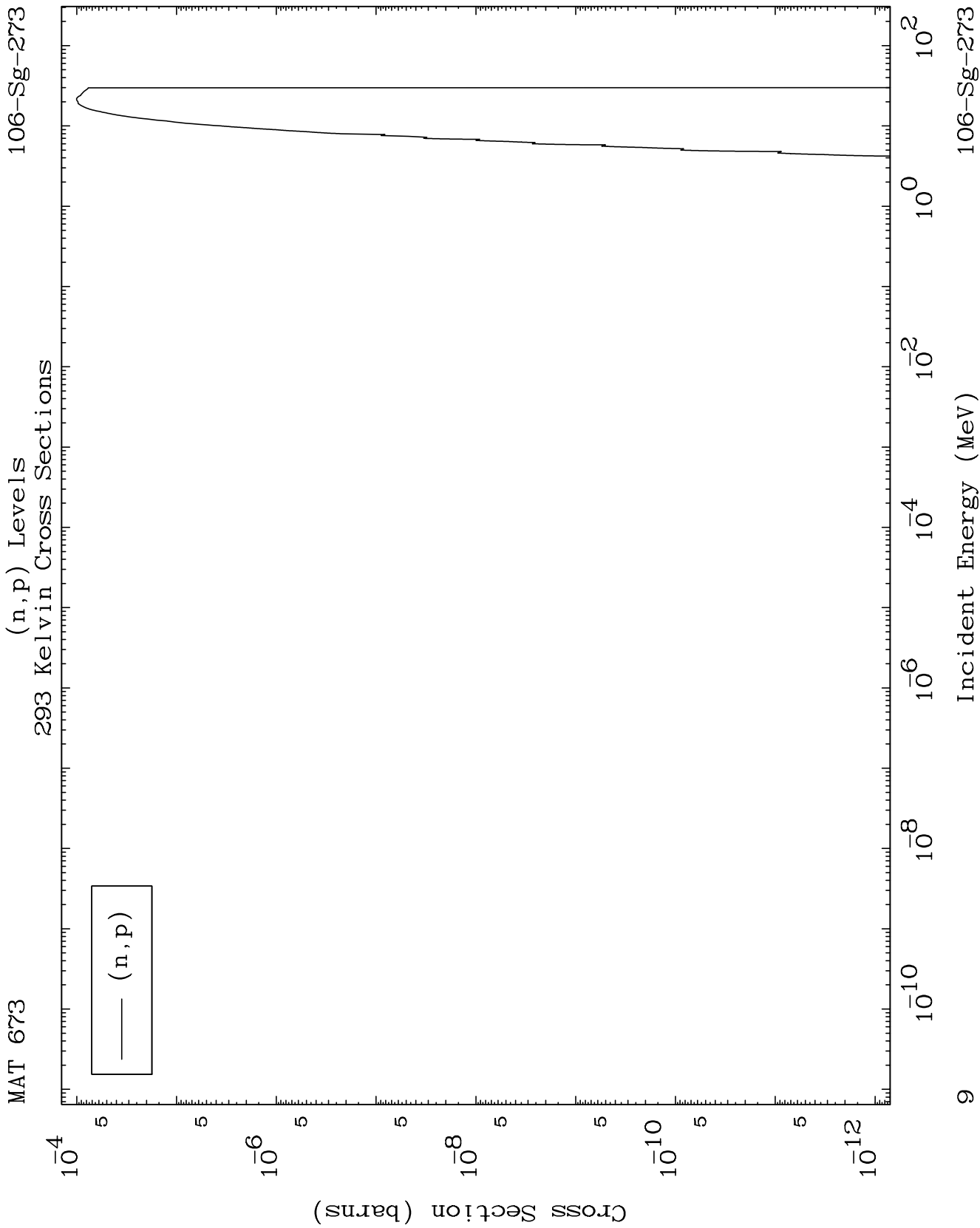
(n,n') Level
293 Kelvin Cross Sections



106-Sg-273

Incident Energy (MeV)

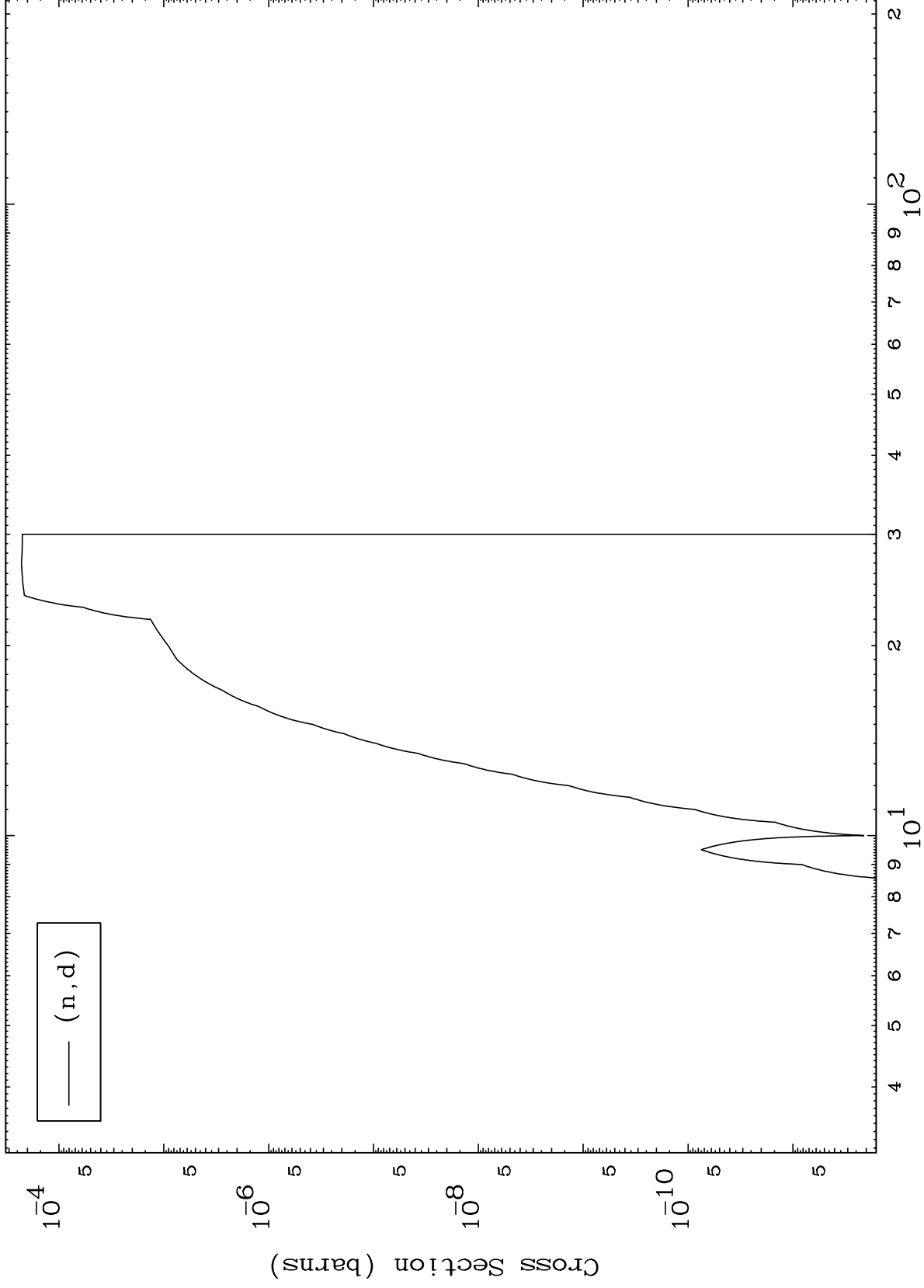
8



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(n,d) Levels
293 Kelvin Cross Sections

106-Sg-273



10

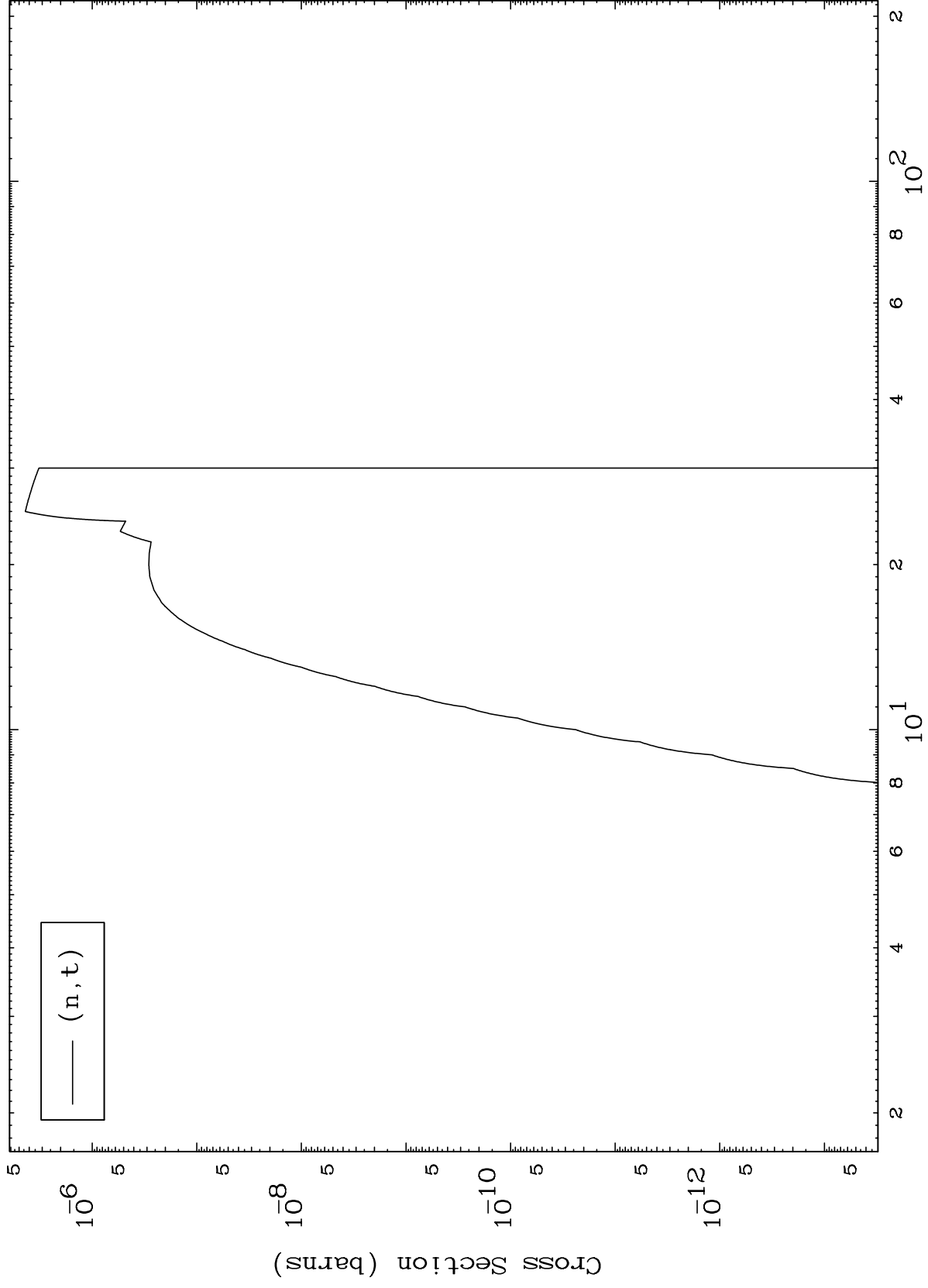
Incident Energy (MeV)

106-Sg-273

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(n,t) Levels
293 Kelvin Cross Sections

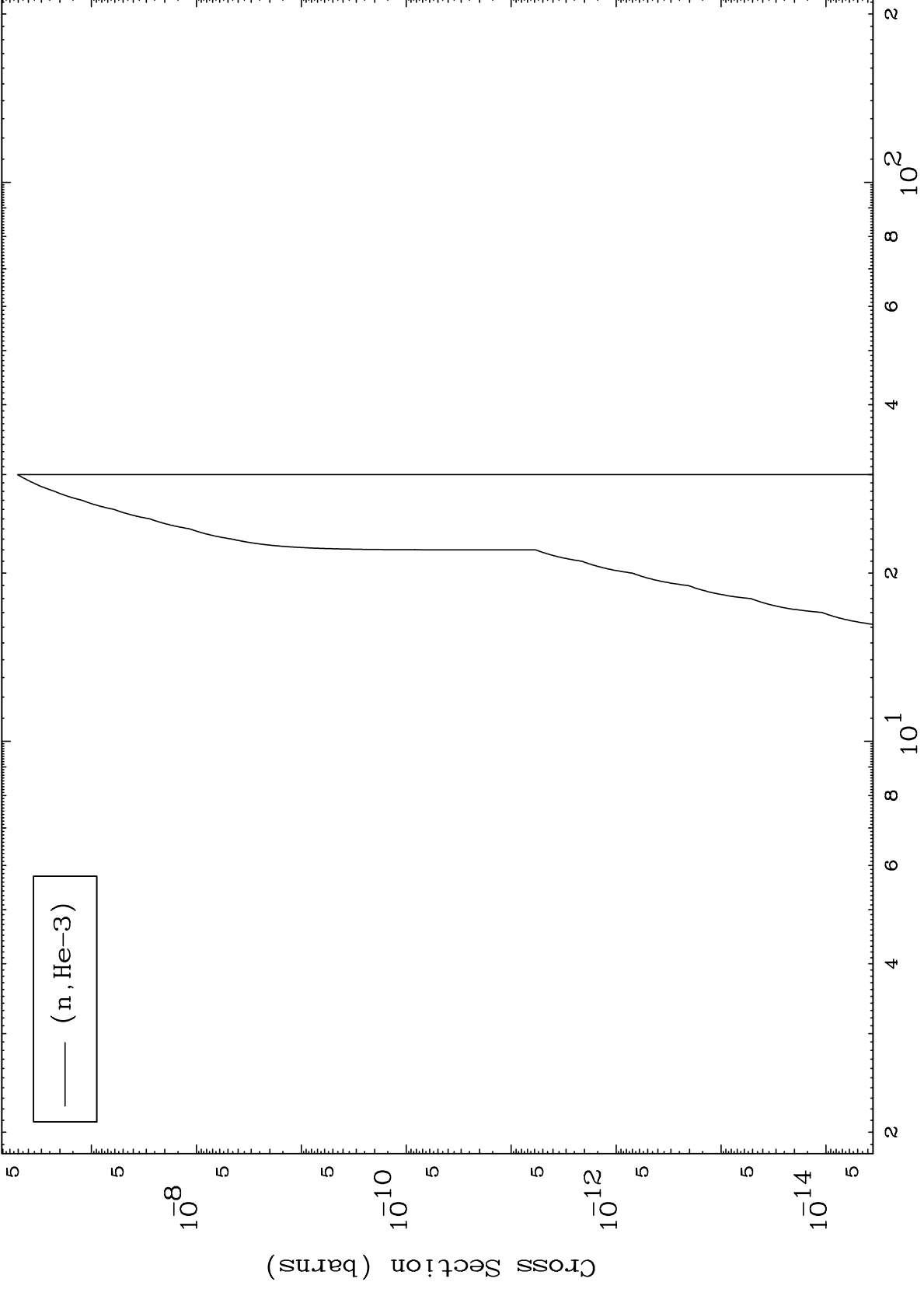
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(n,He3) Levels
293 Kelvin Cross Sections

106-Sg-273



12

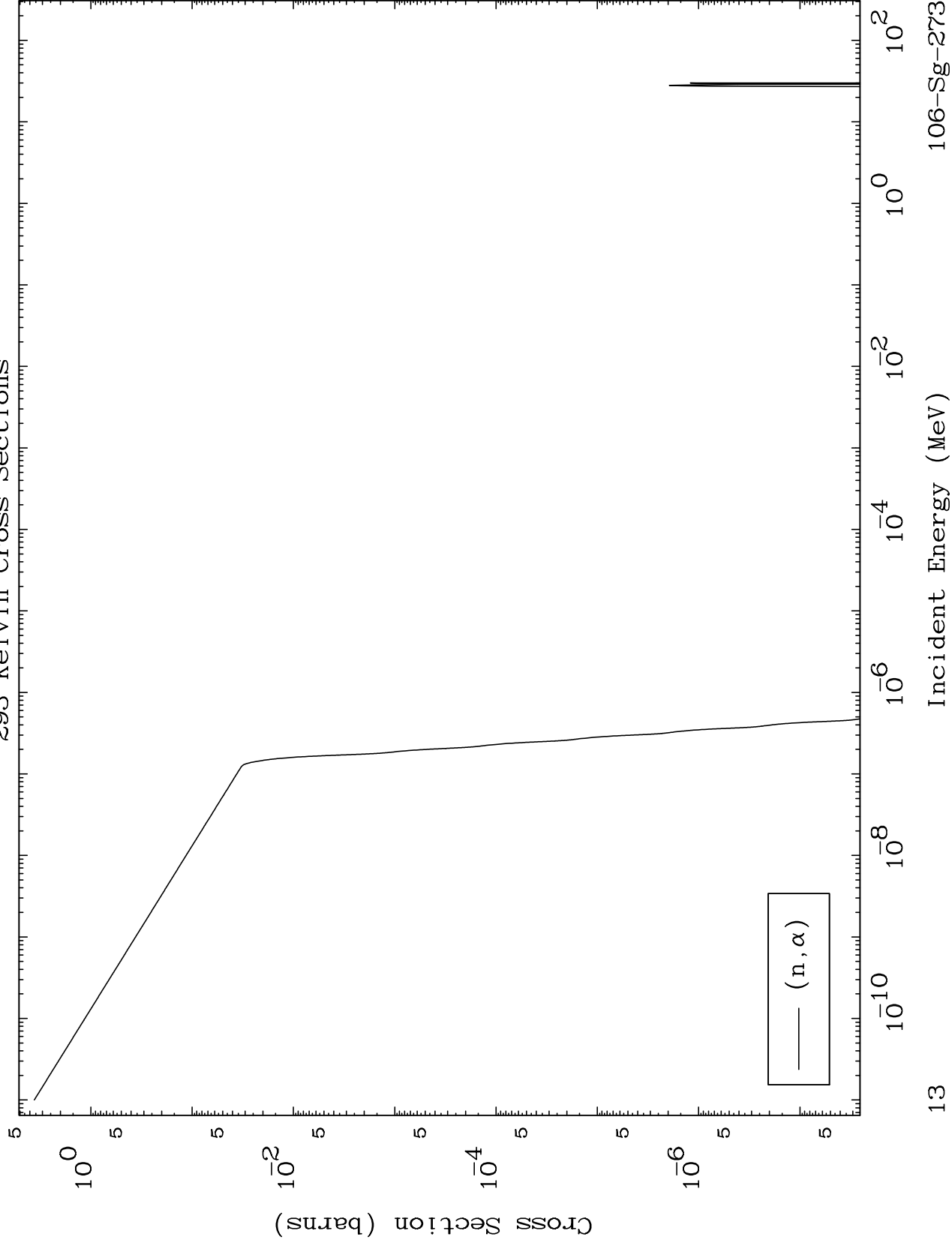
Incident Energy (MeV)

106-Sg-273

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(n,α) Levels
293 Kelvin Cross Sections

106-Sg-273

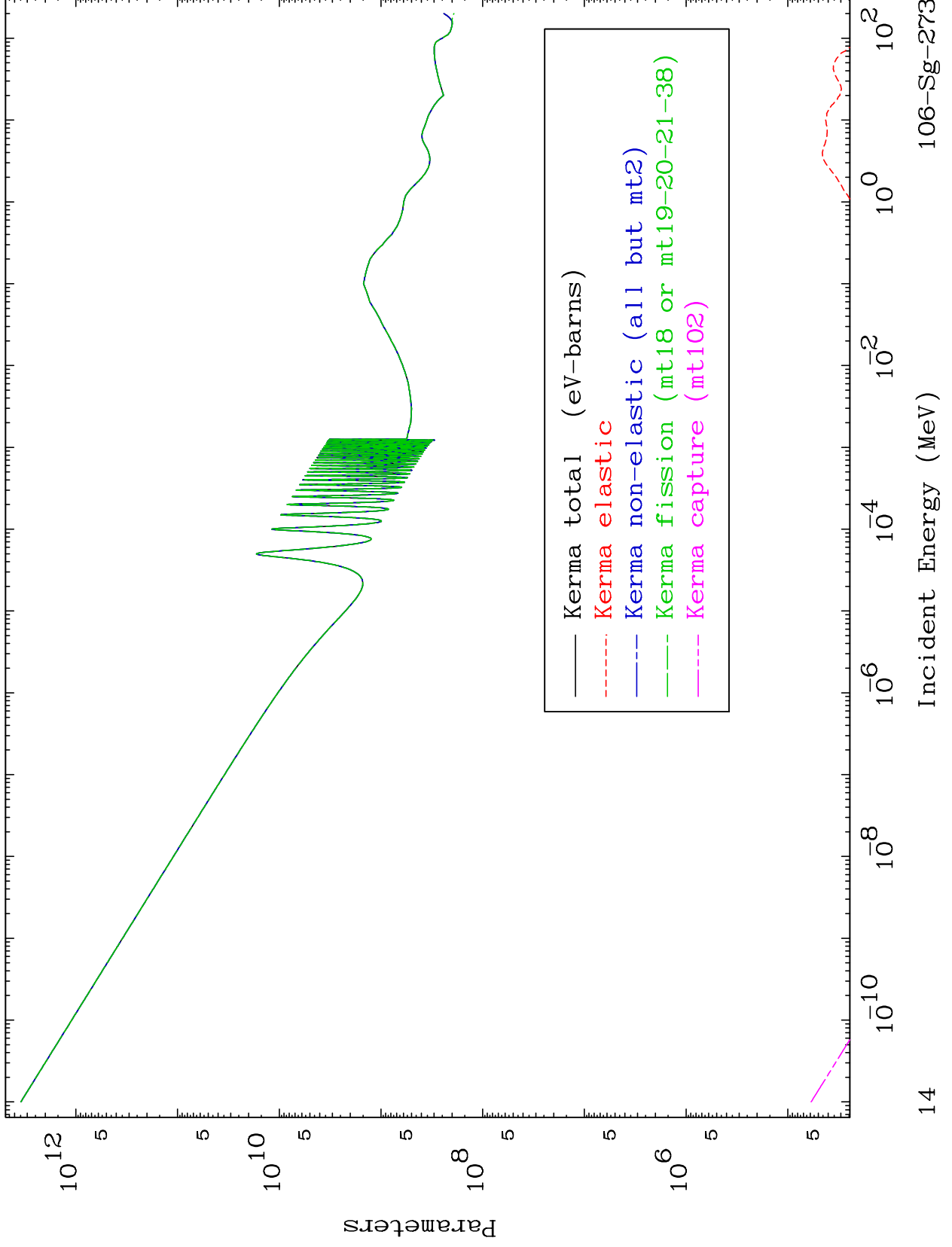


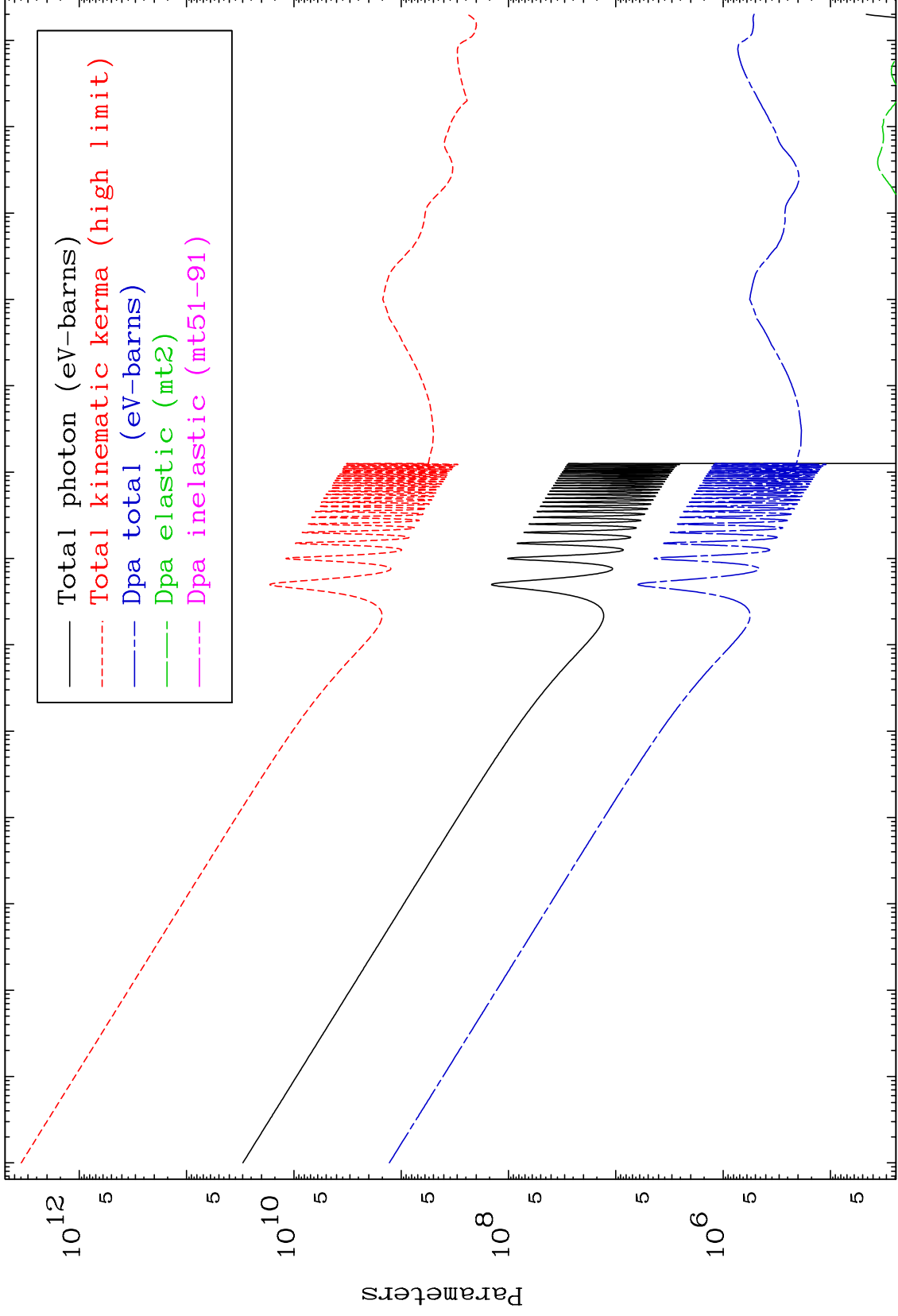
13

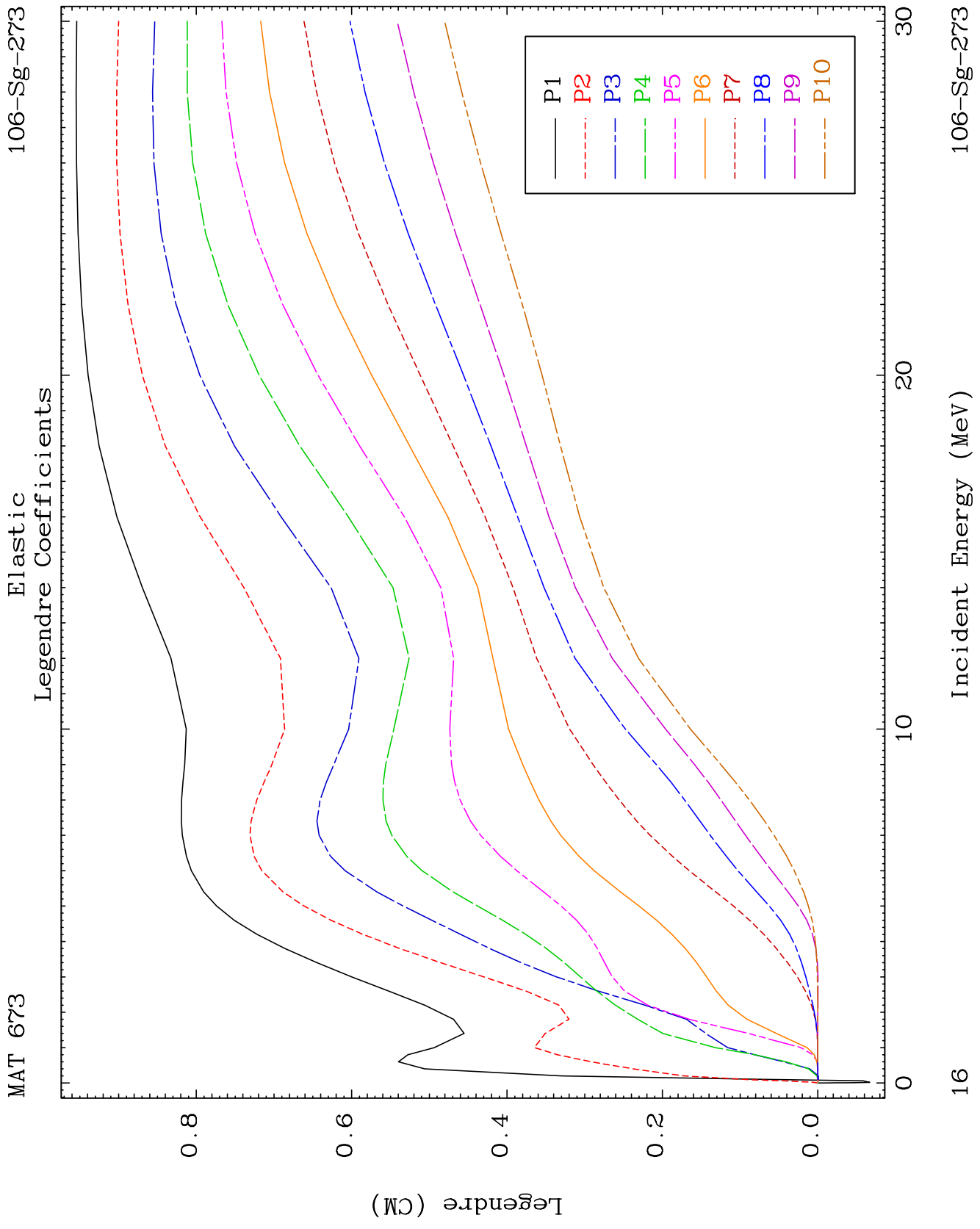
MAT 673

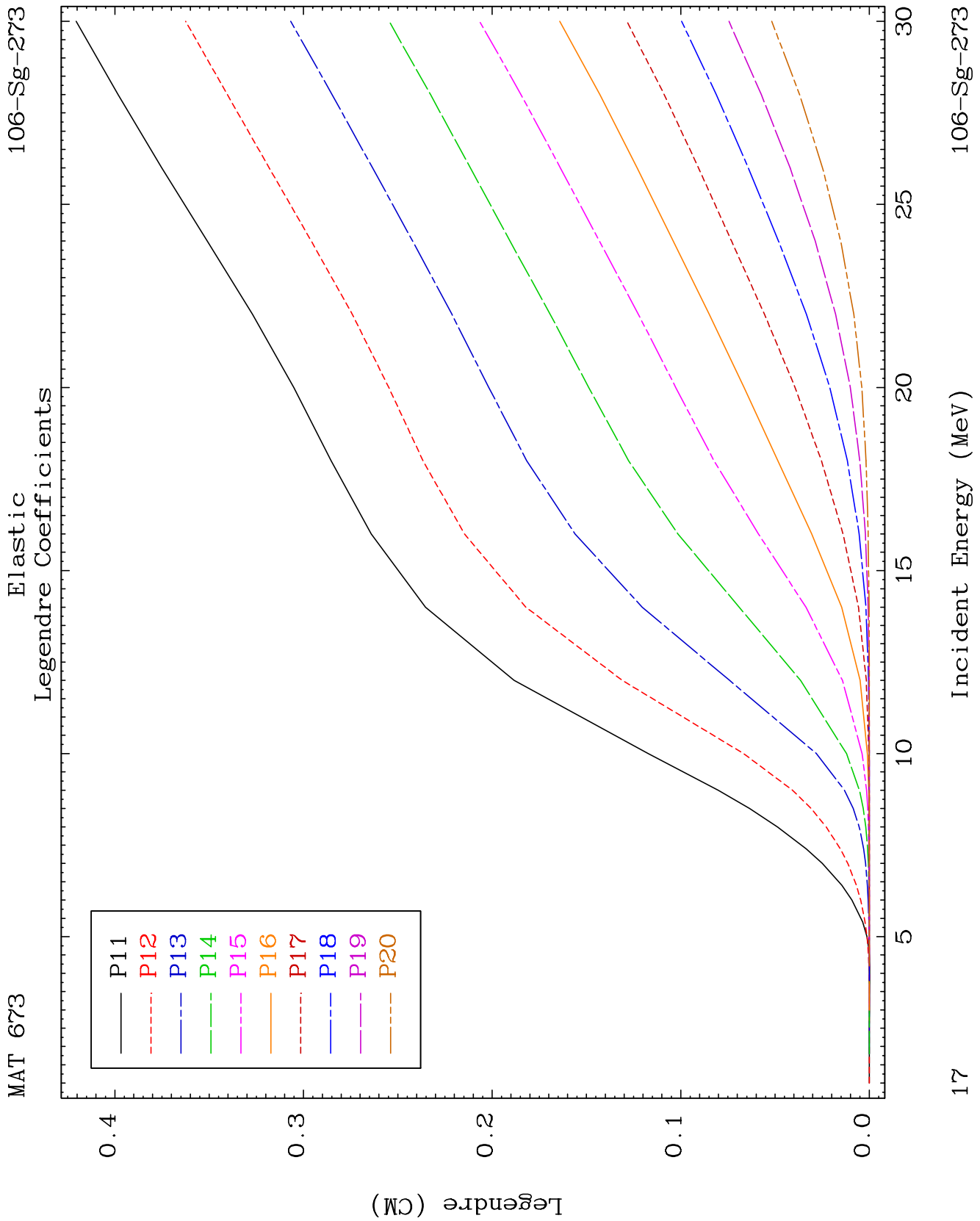
Energy Release
Parameters

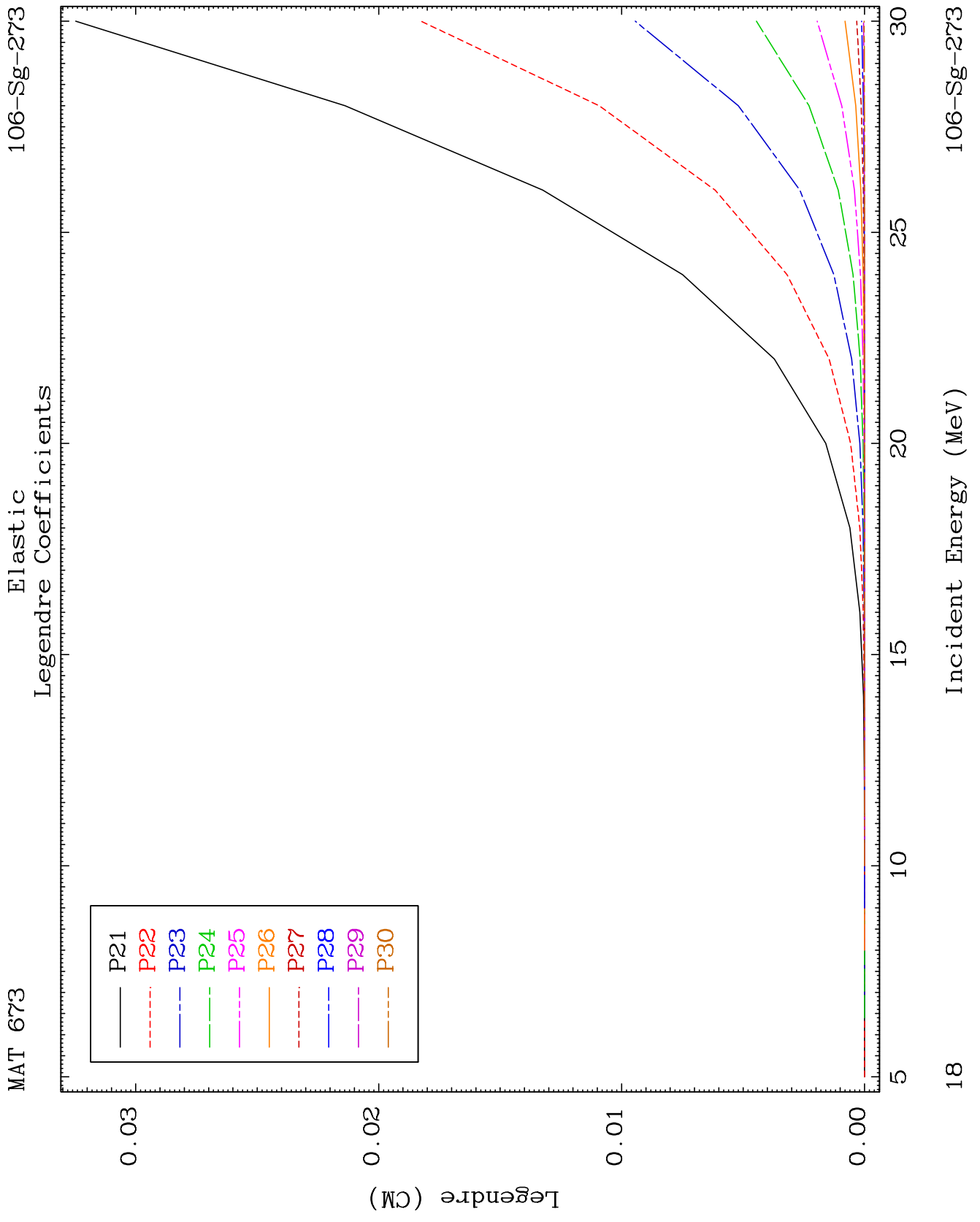
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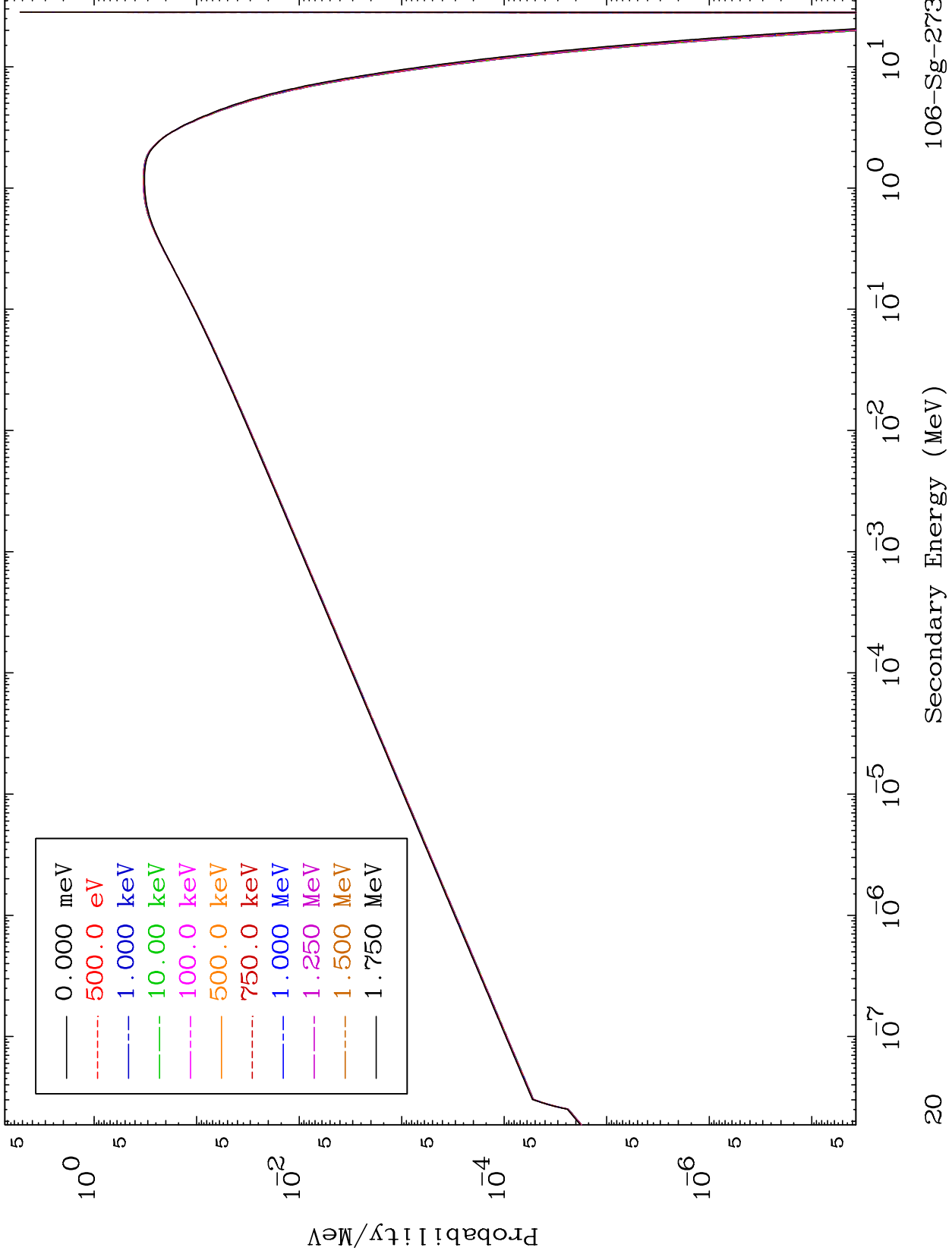




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Fission Energy Distribution

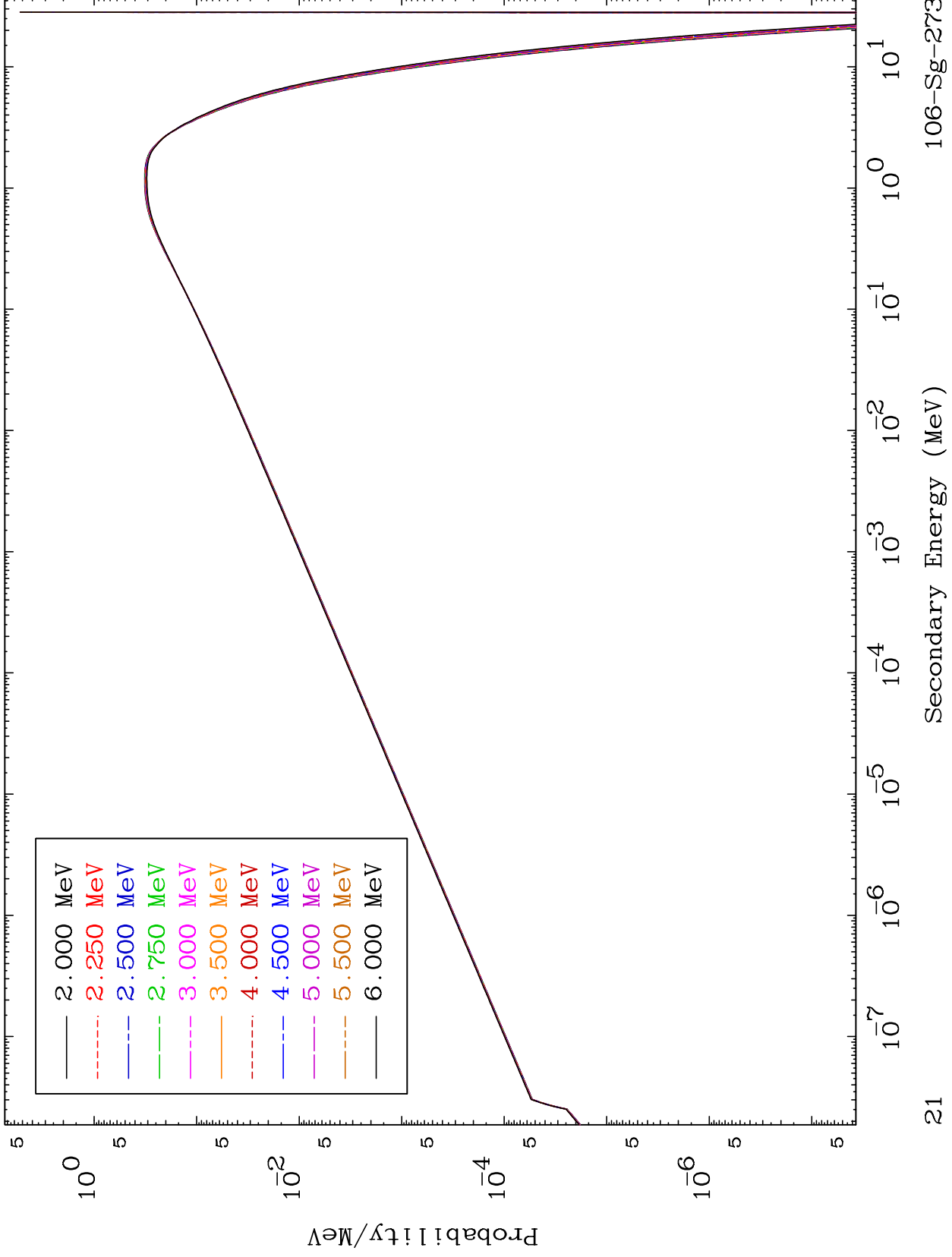
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Fission Energy Distribution

106-Sg-273

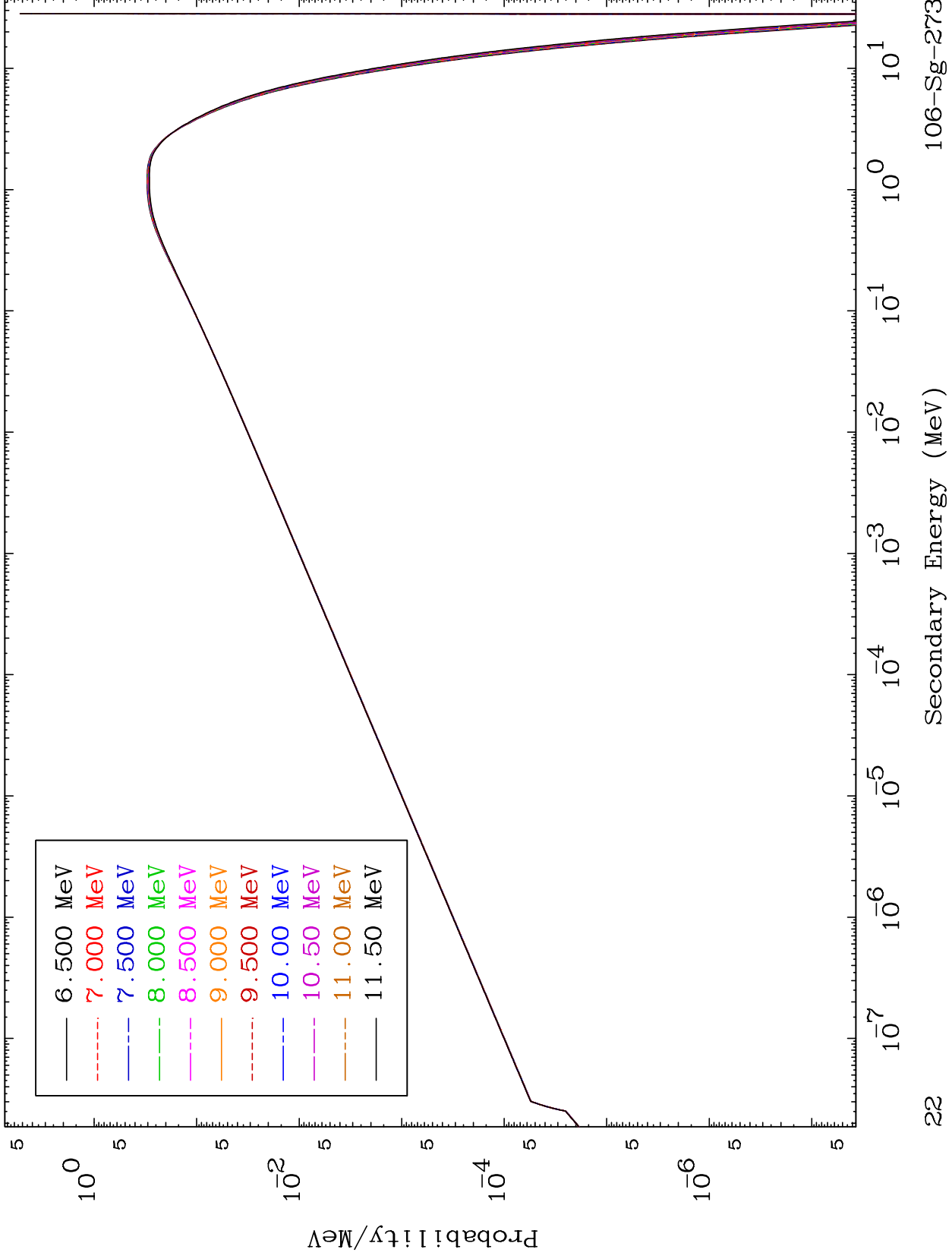


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Fission
Energy Distribution

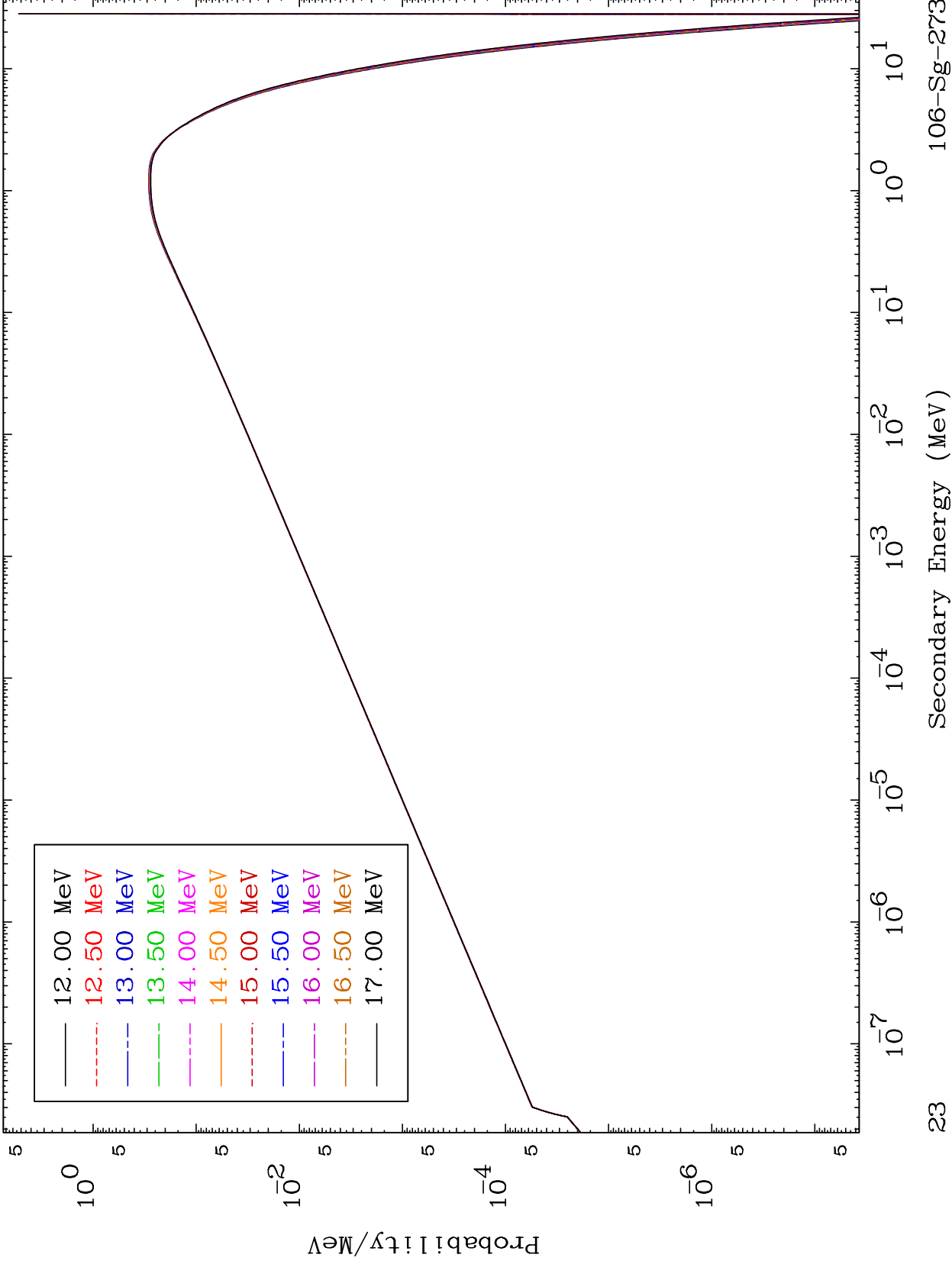
106-Sg-273



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Fission Energy Distribution

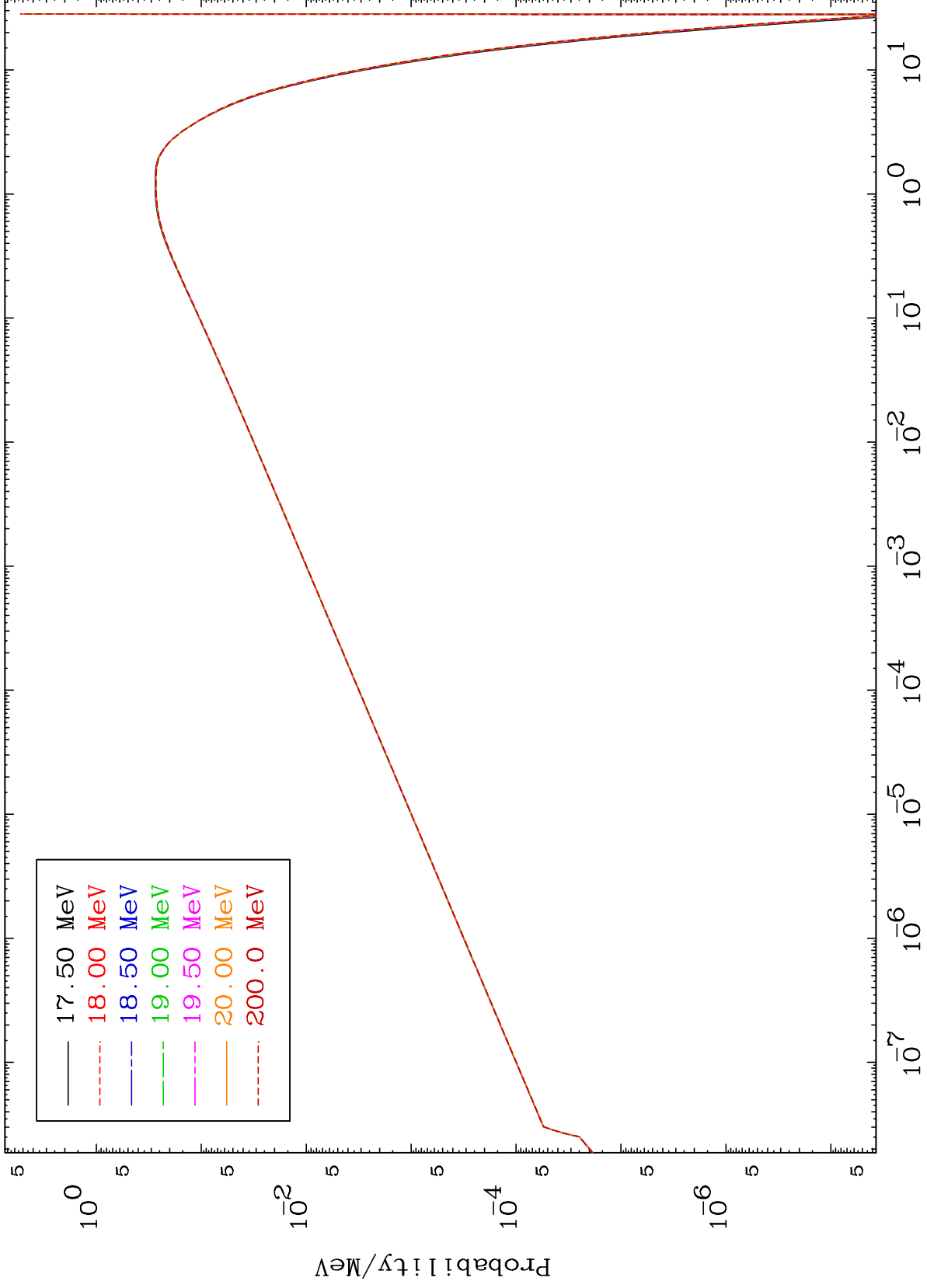
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Fission Energy Distribution

106-Sg-273



24

Secondary Energy (MeV)

106-Sg-273