

Program EVALPLOT
(Version 2015-2)

by

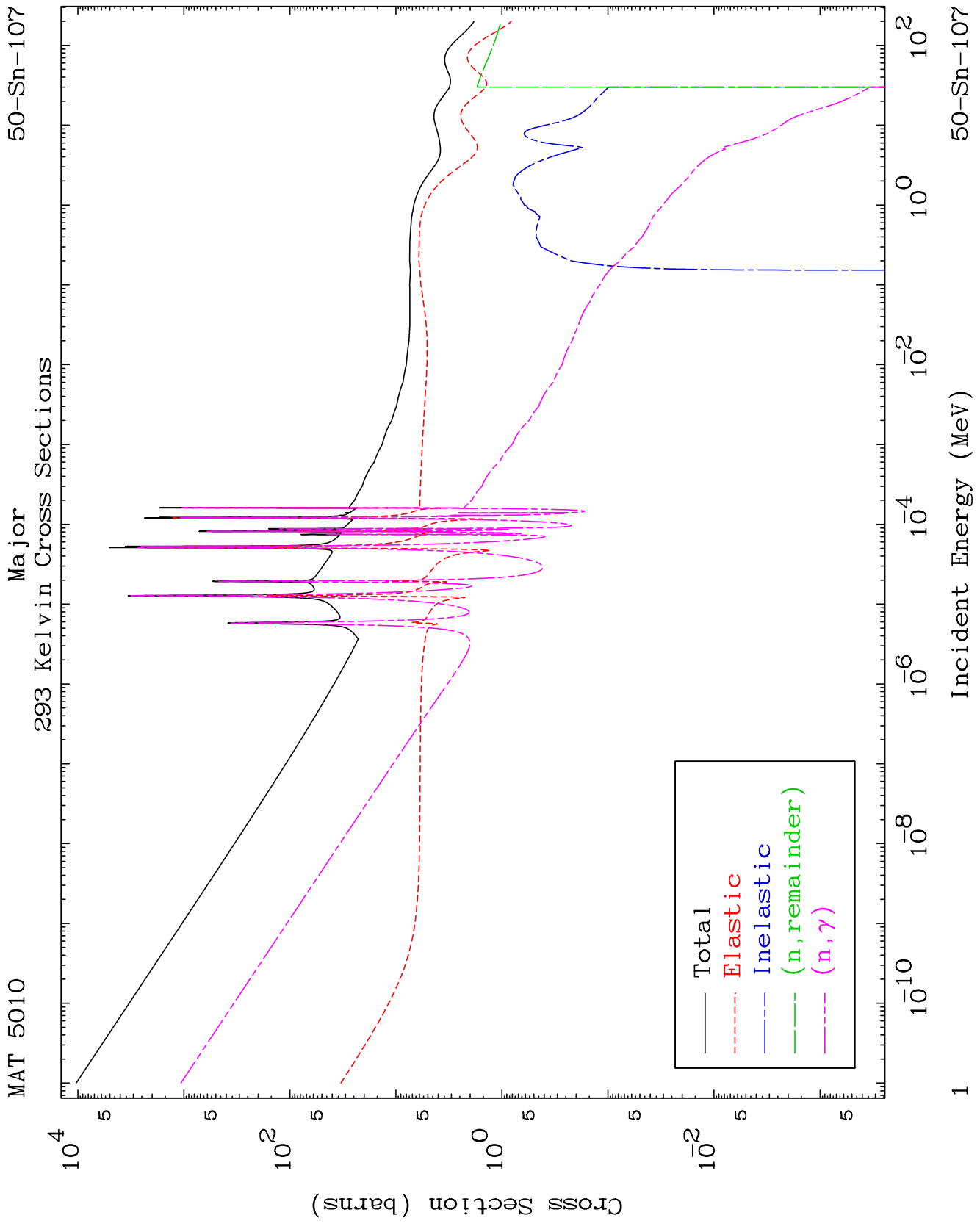
Dermott E. Cullen
(Present Contact Information)

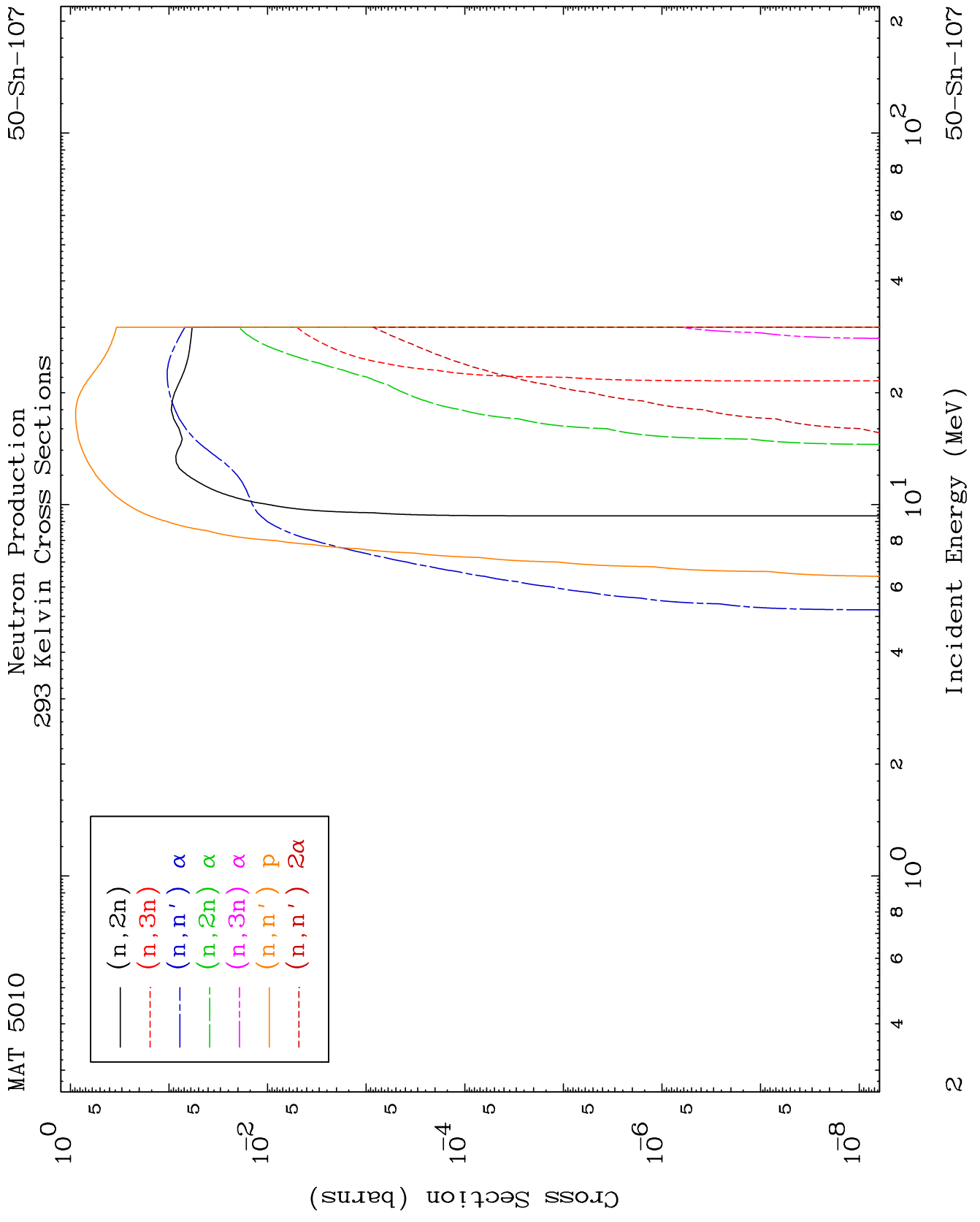
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Press Mouse Button to Start

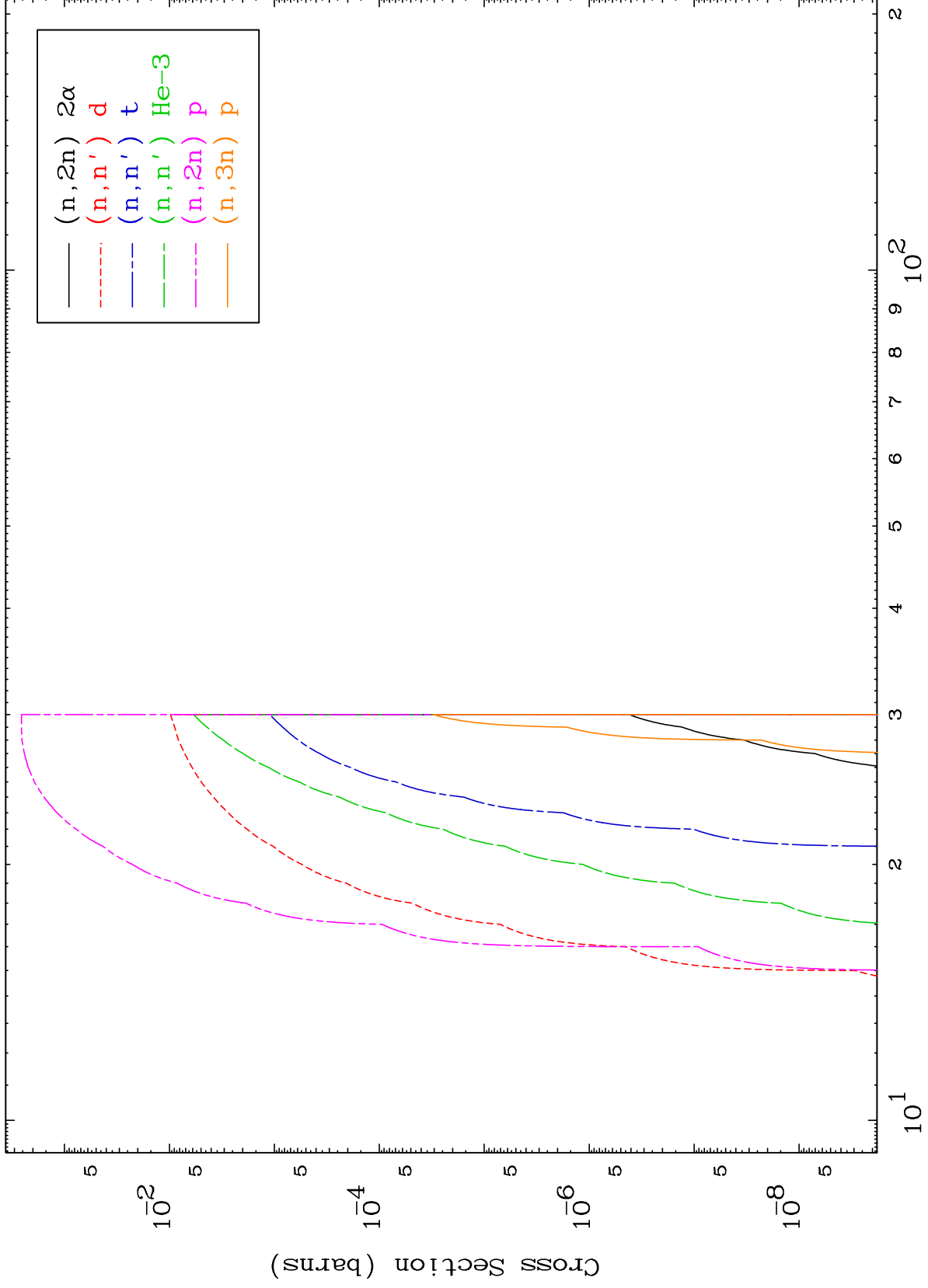




MAT 5010

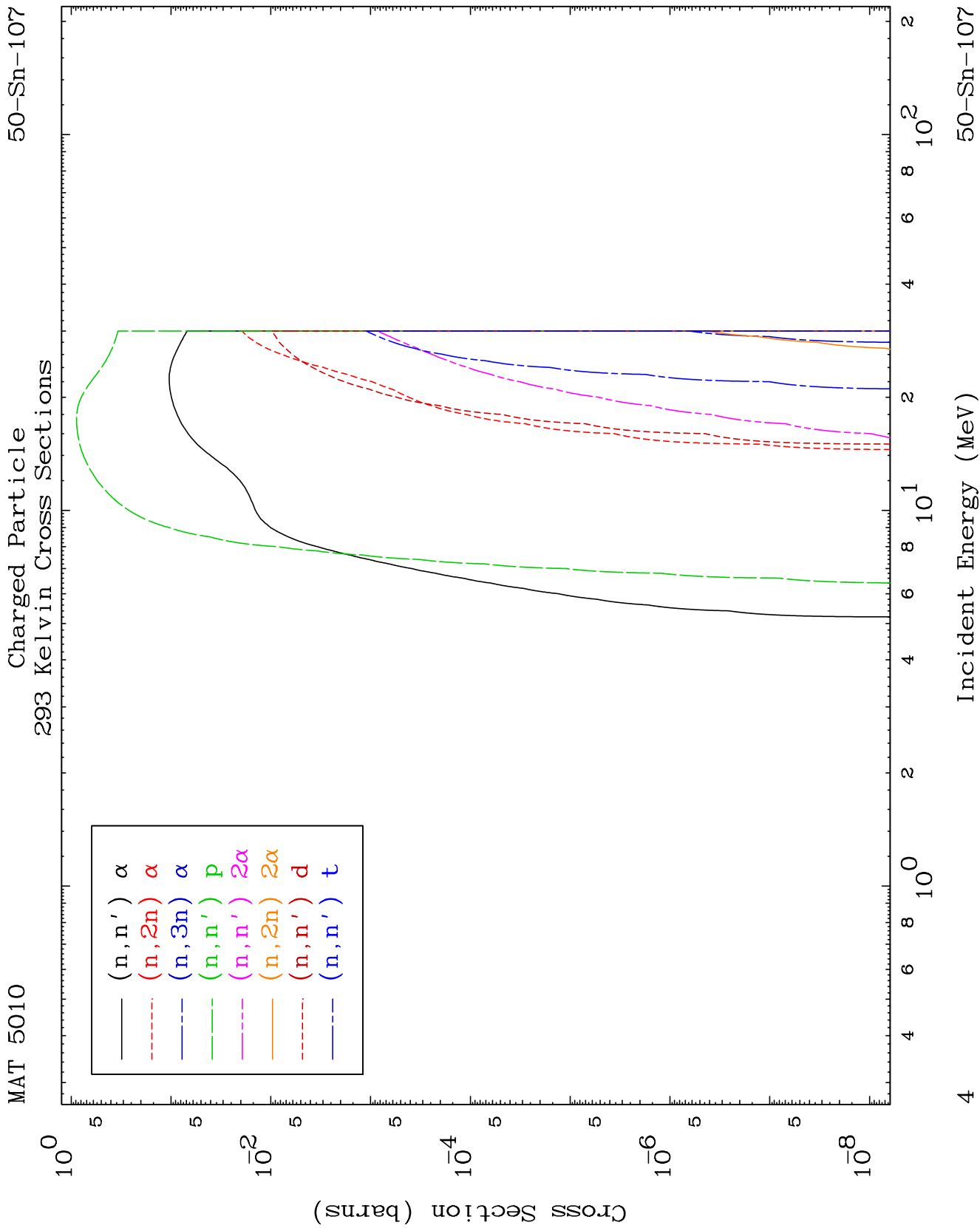
Neutron Production
293 Kelvin Cross Sections

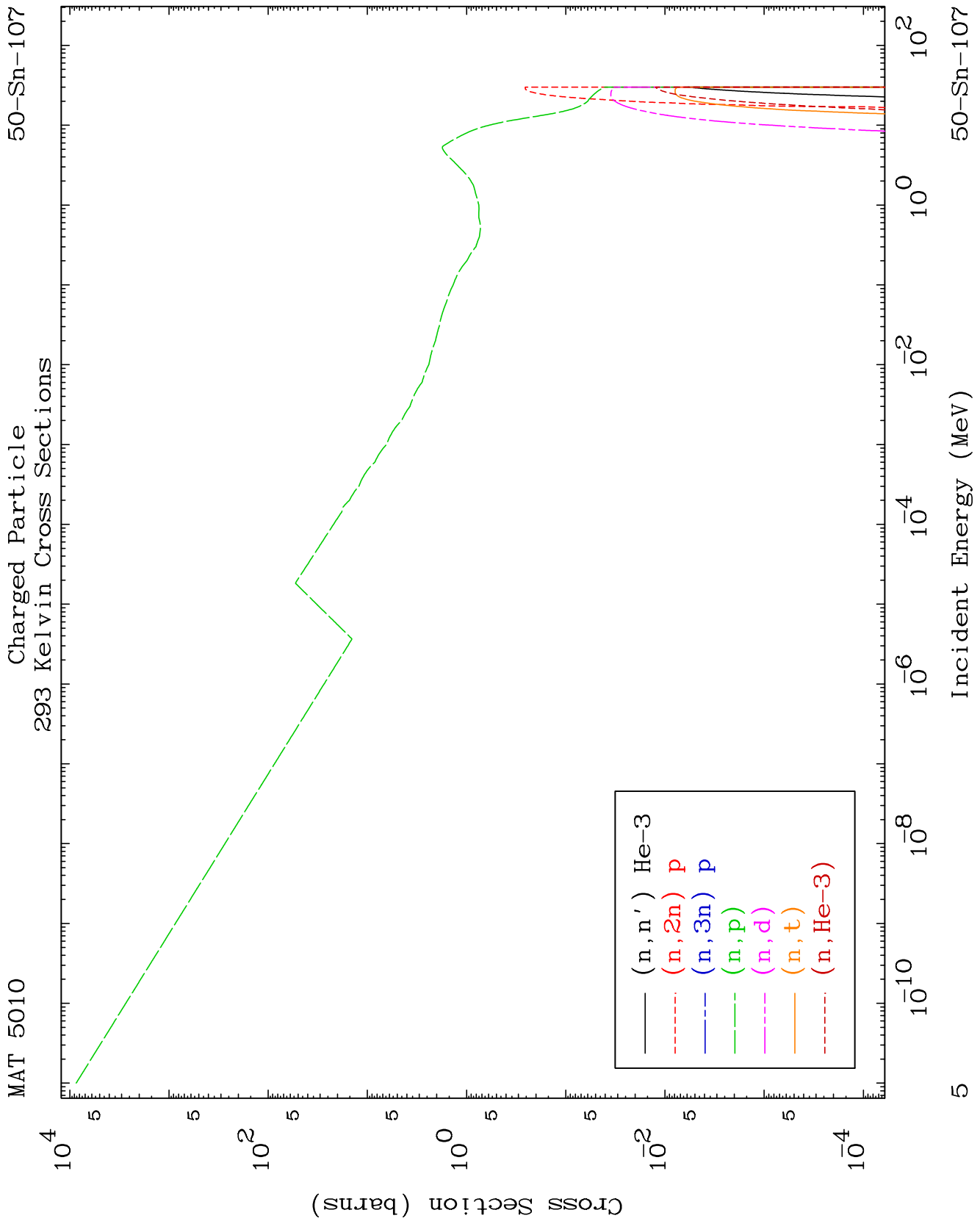
50-Sn-107



Incident Energy (MeV)

50-Sn-107

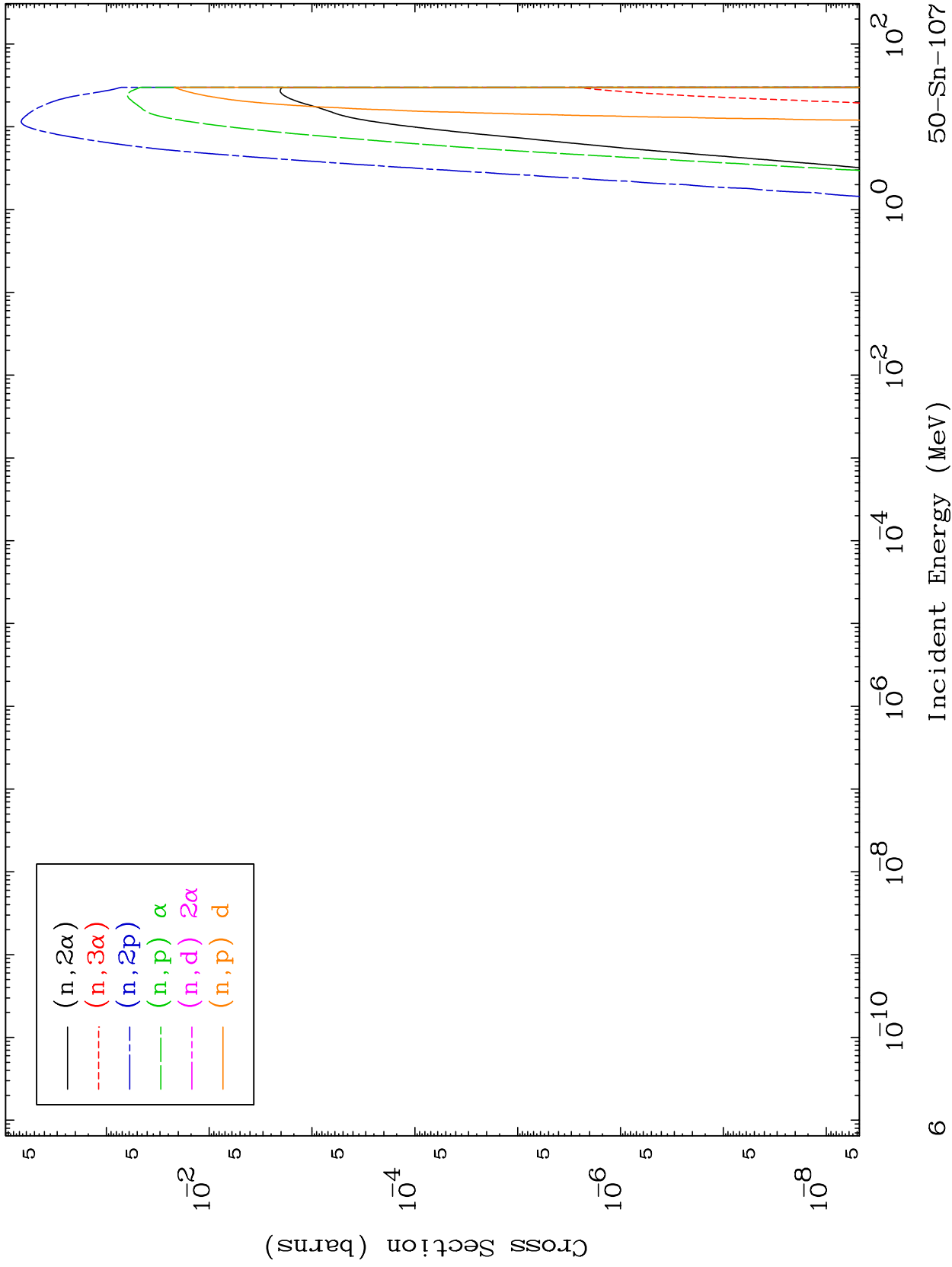


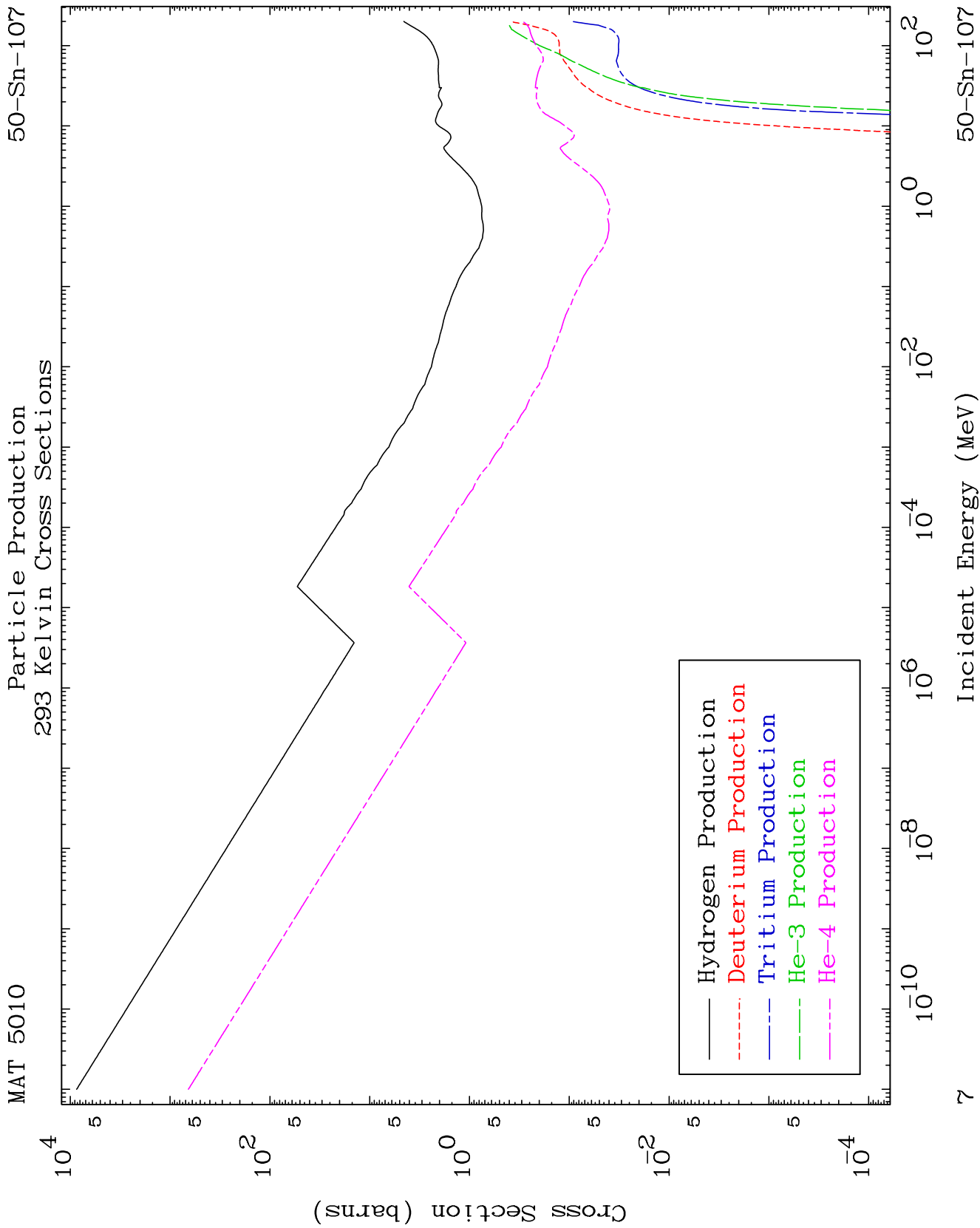


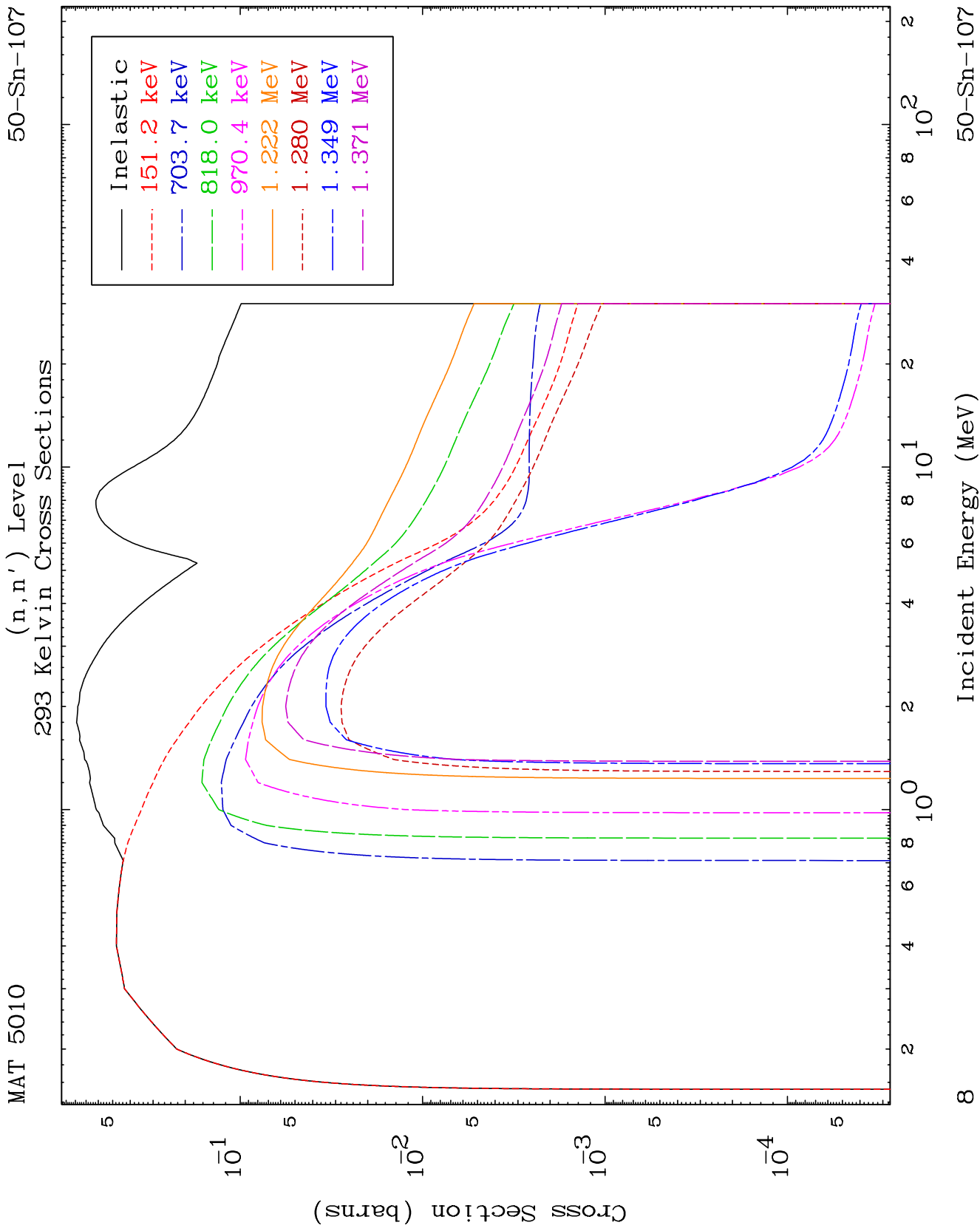
MAT 5010

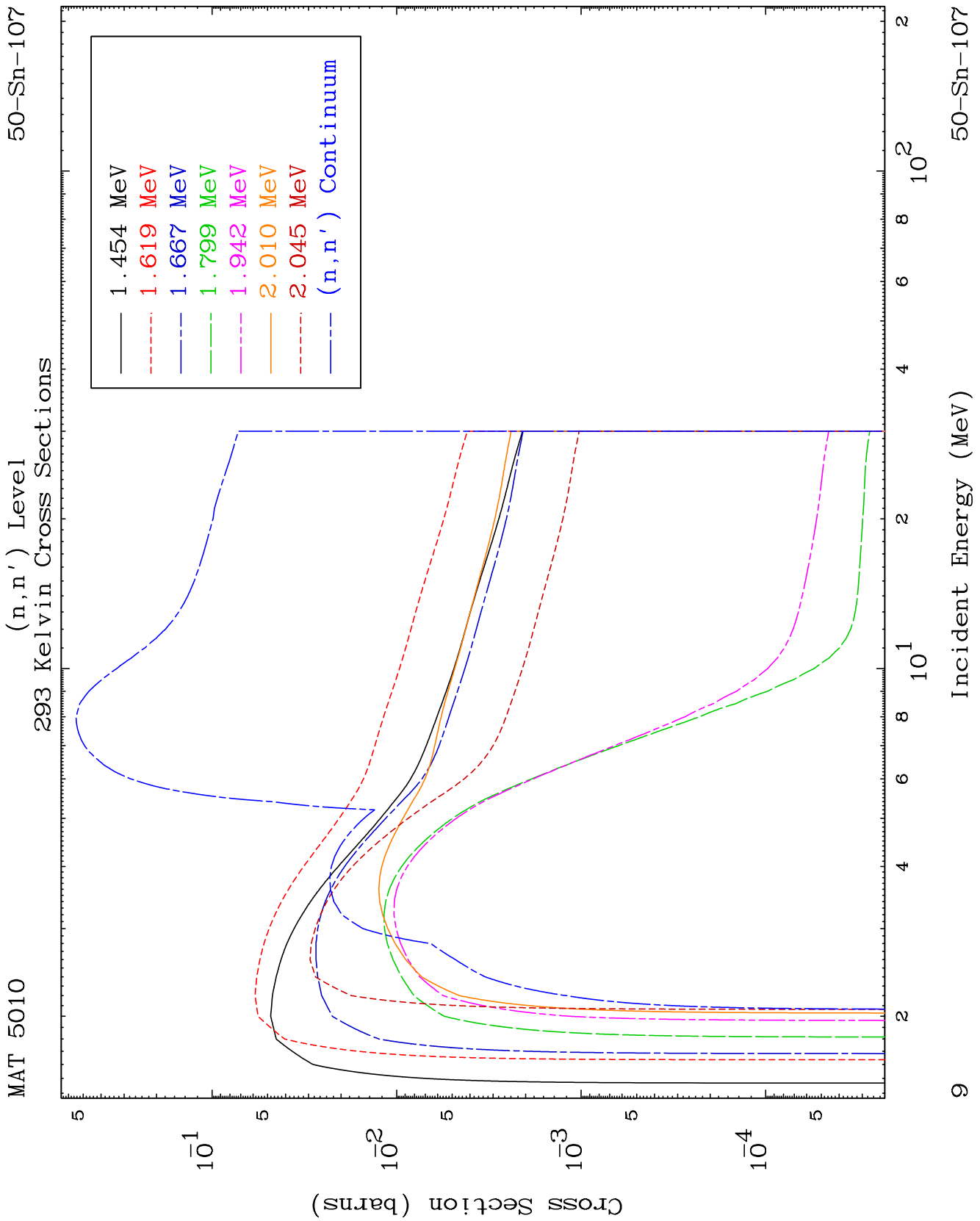
Charged Particle
293 Kelvin Cross Sections

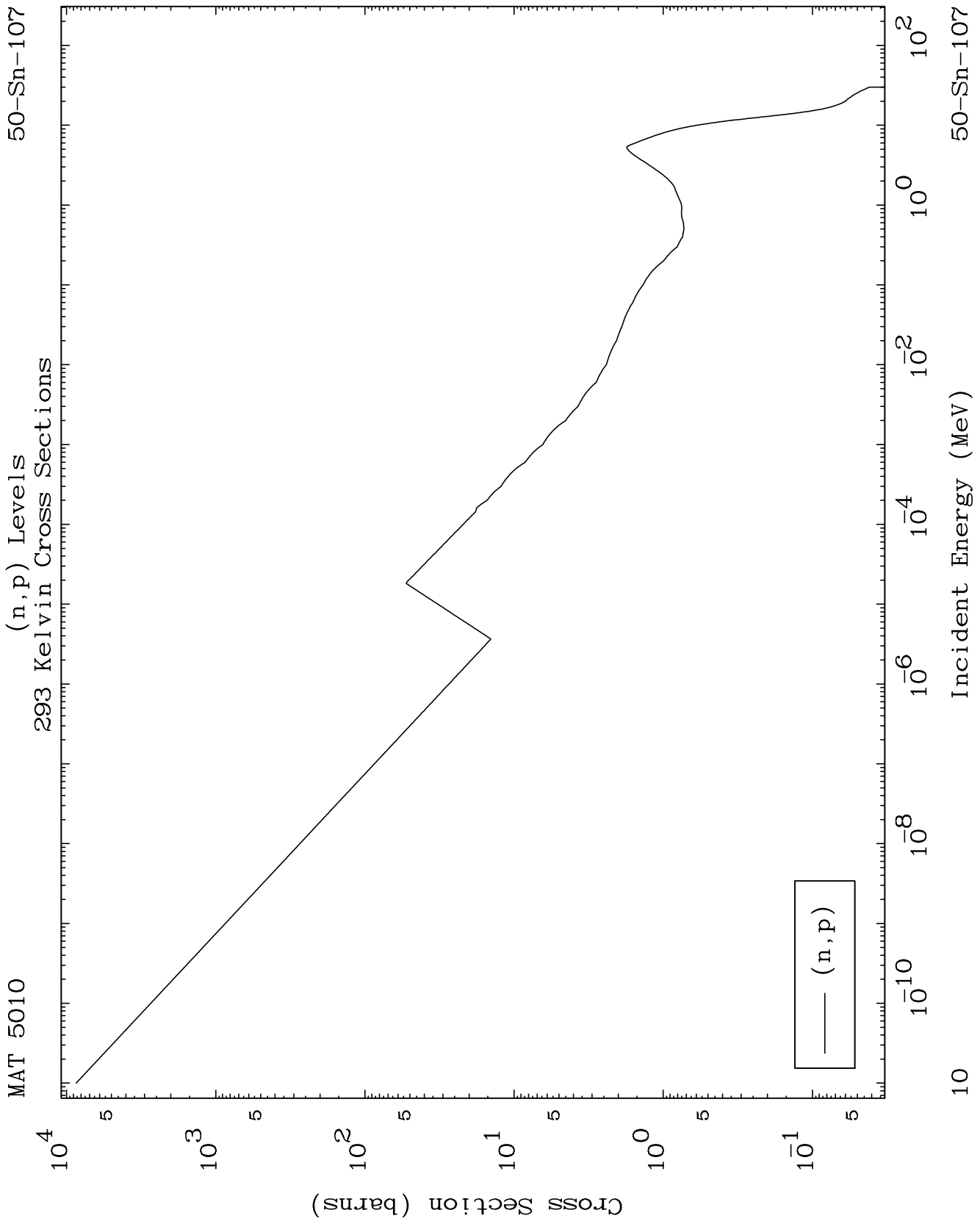
50-Sn-107







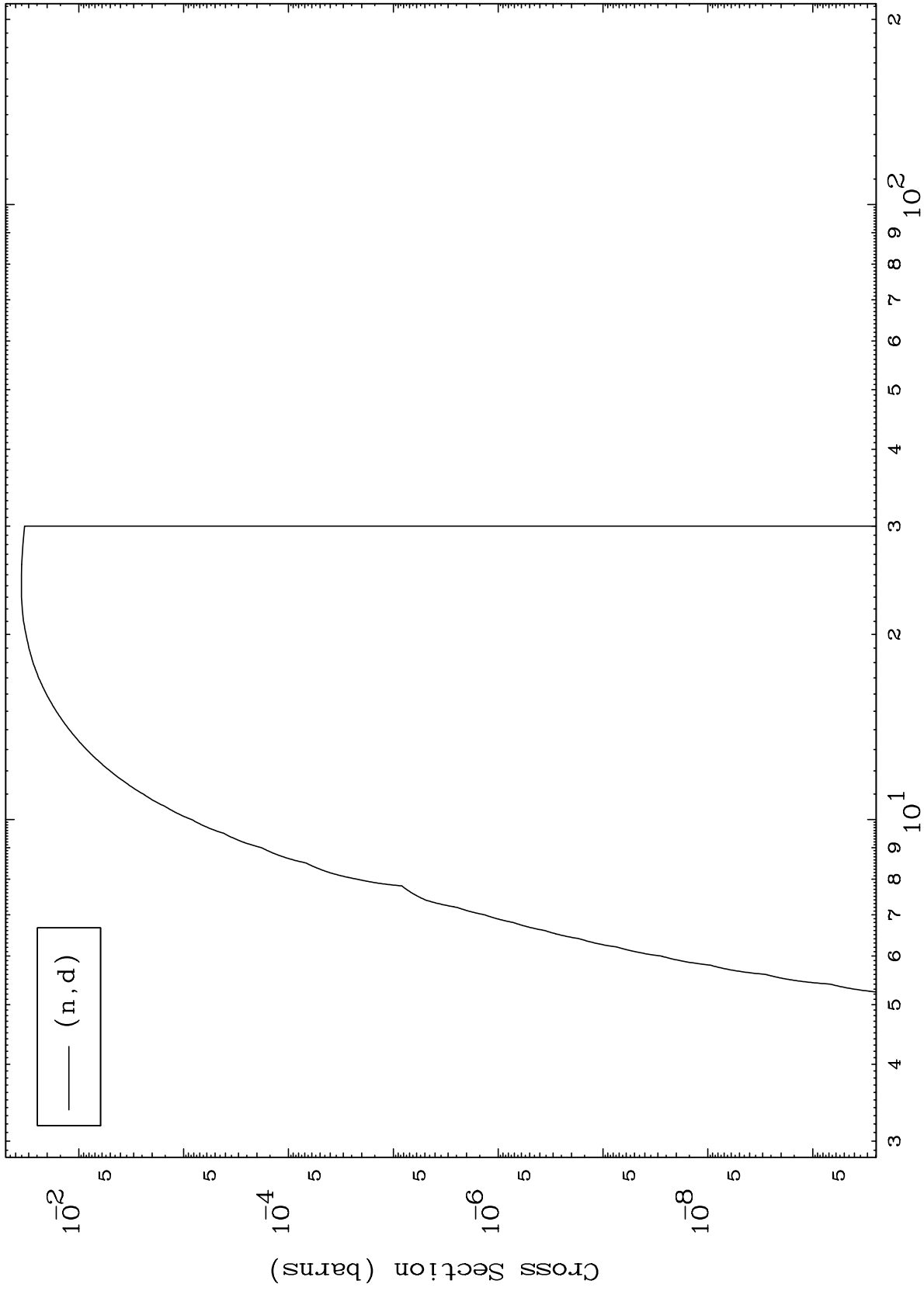




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(n,d) Levels
293 Kelvin Cross Sections

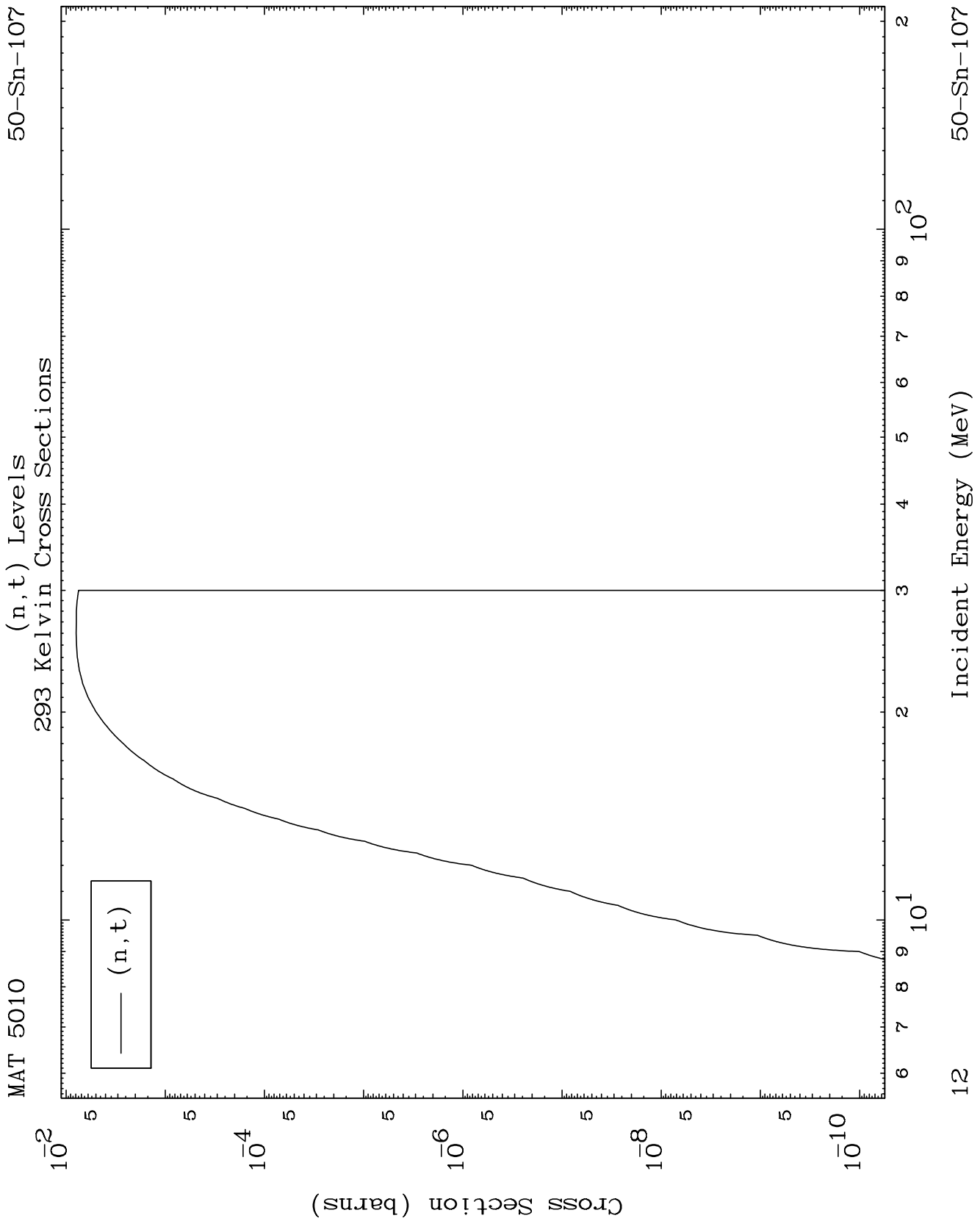
50-Sn-107



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Incident Energy (MeV)

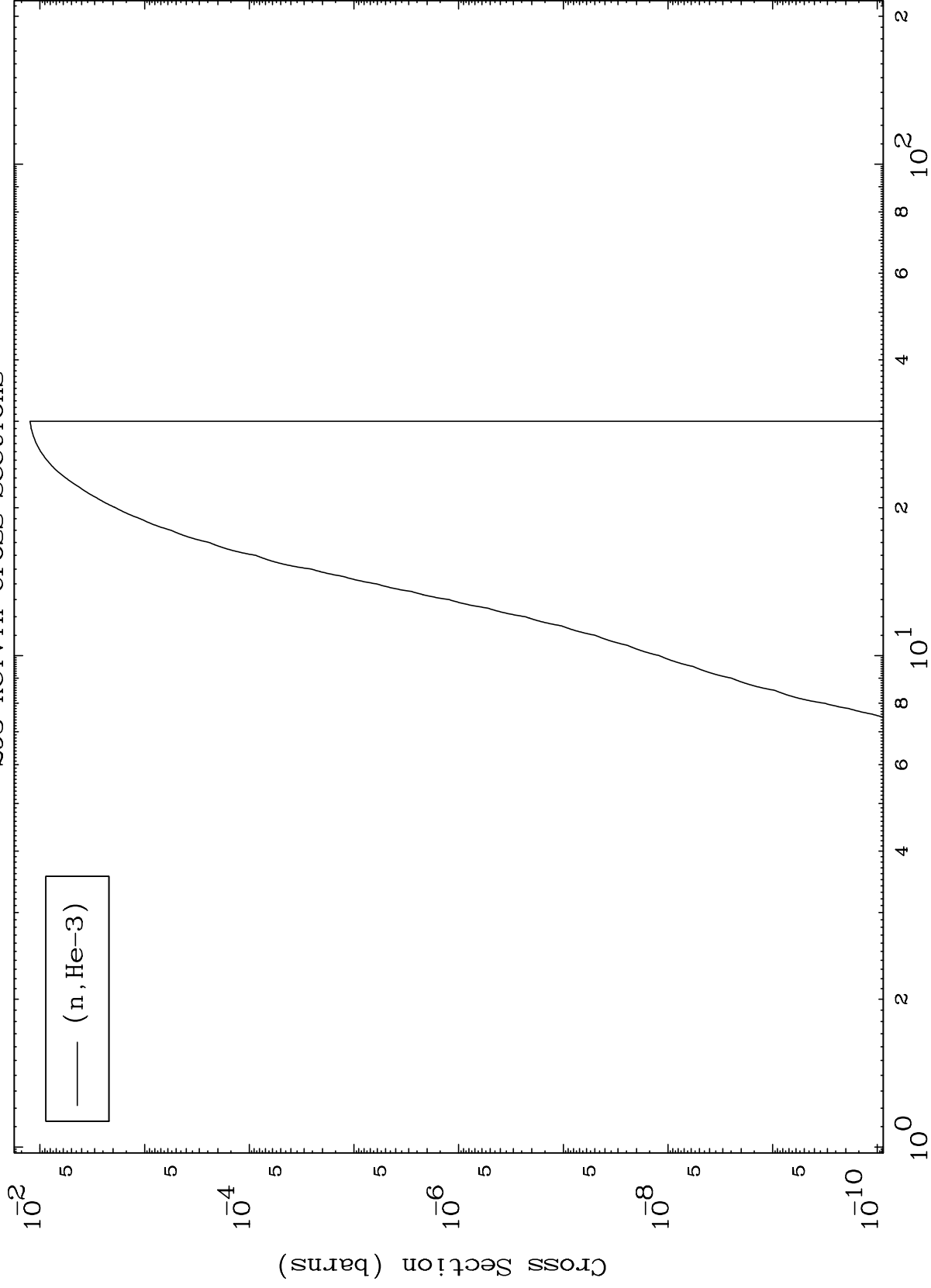
50-Sn-107



MAT 5010

(n,He3) Levels
293 Kelvin Cross Sections

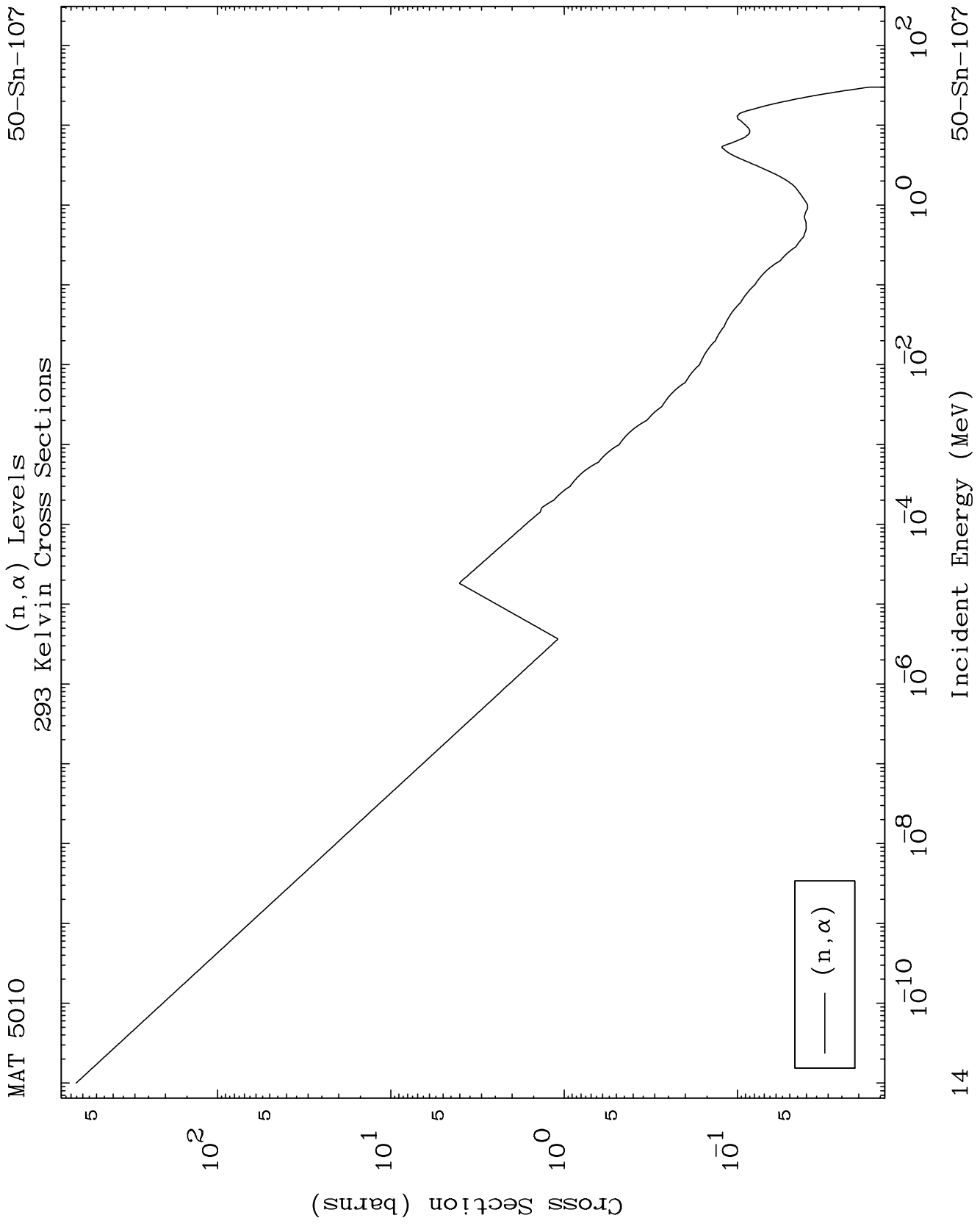
50-Sn-107

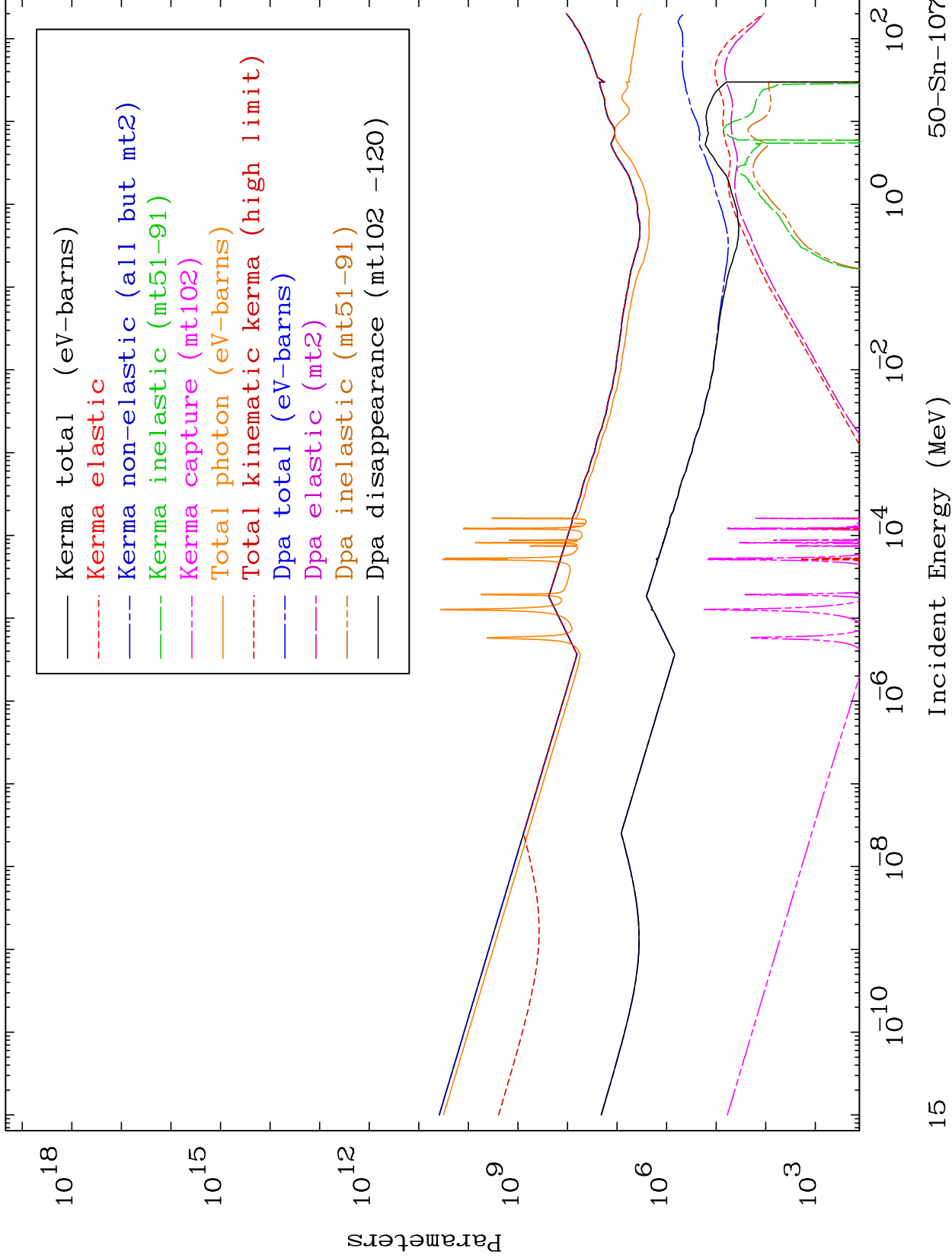


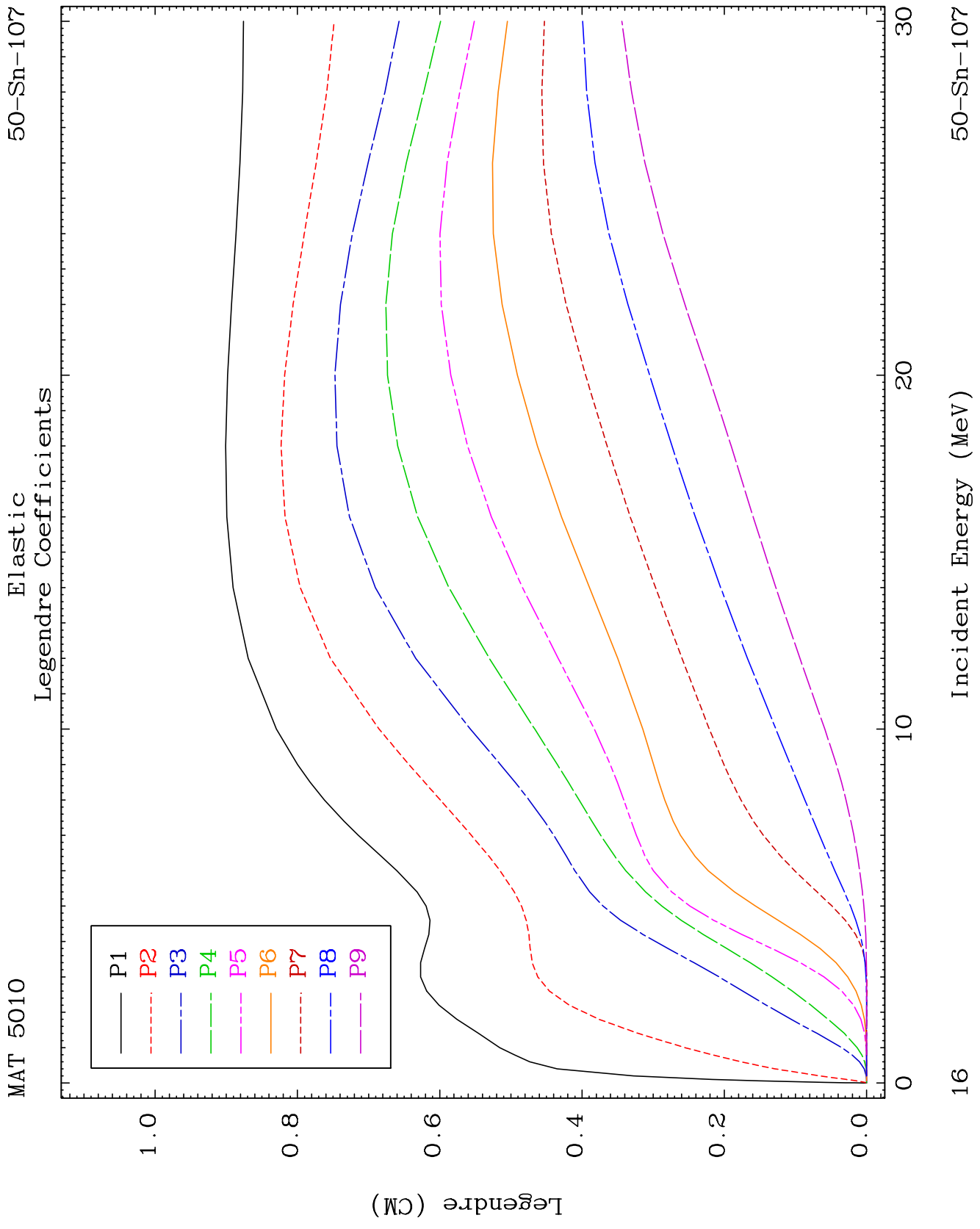
Incident Energy (MeV)

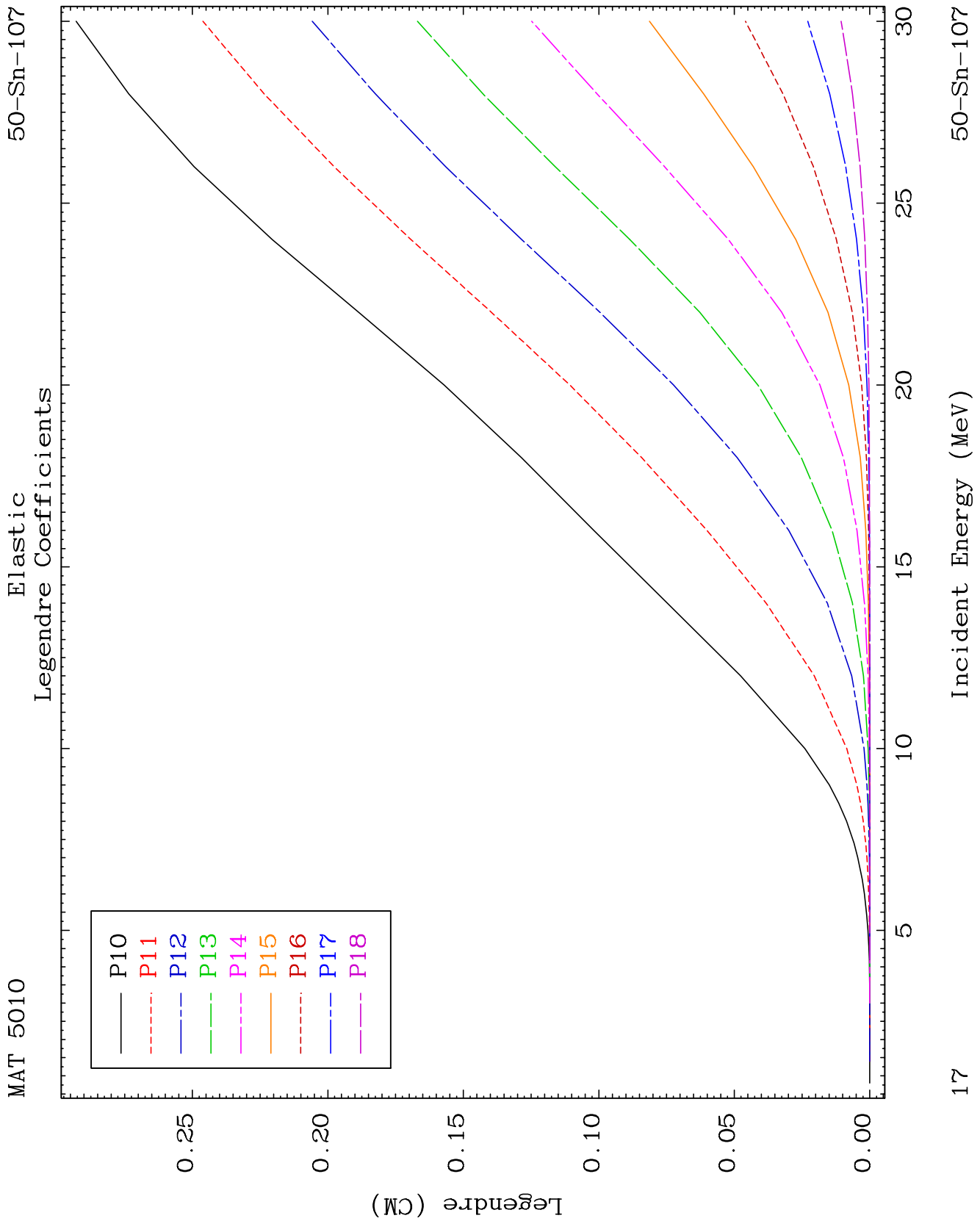
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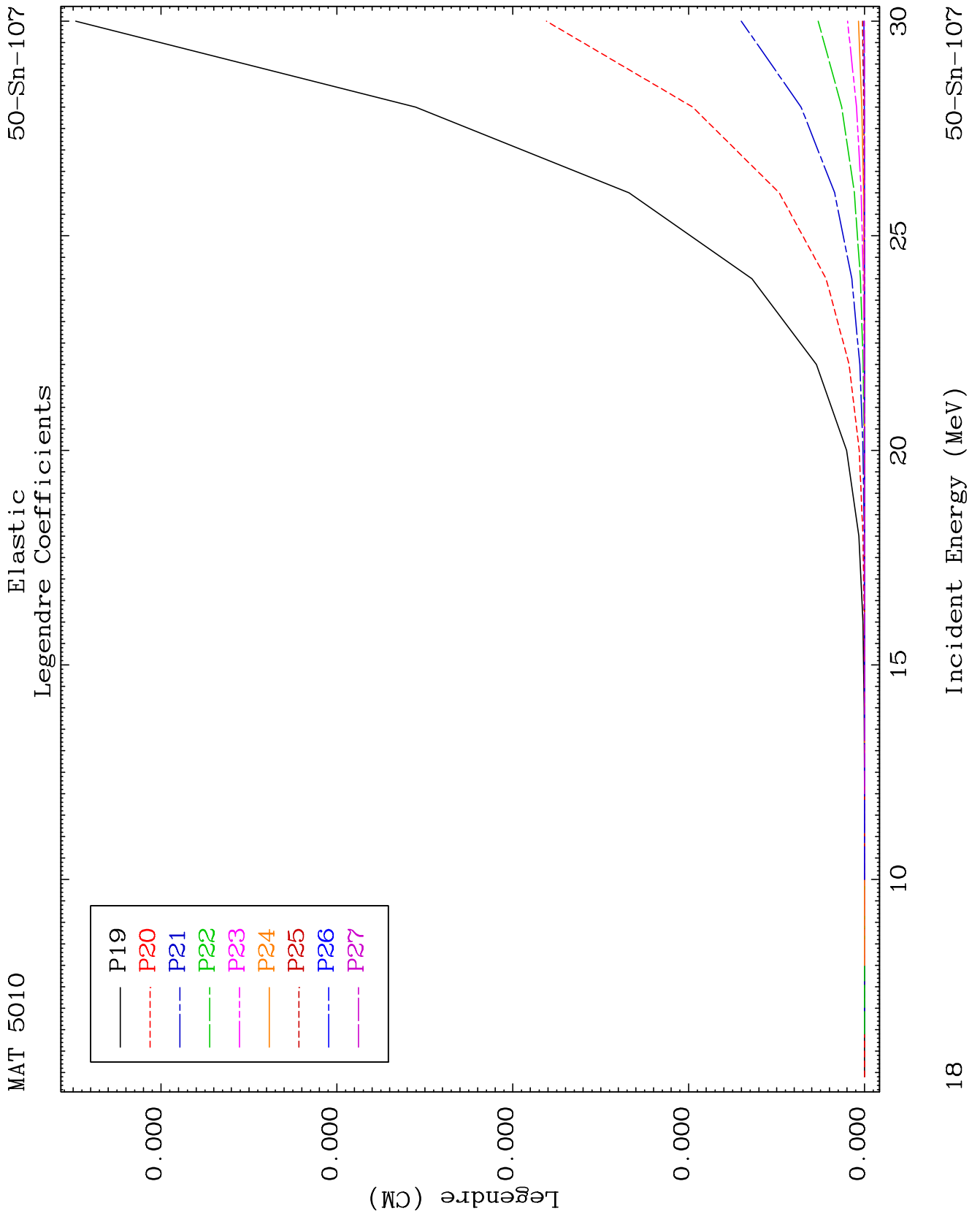
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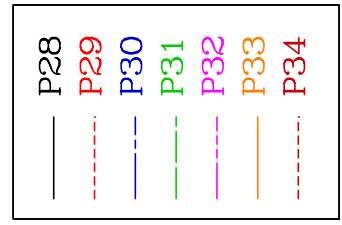




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Elastic Legendre Coefficients

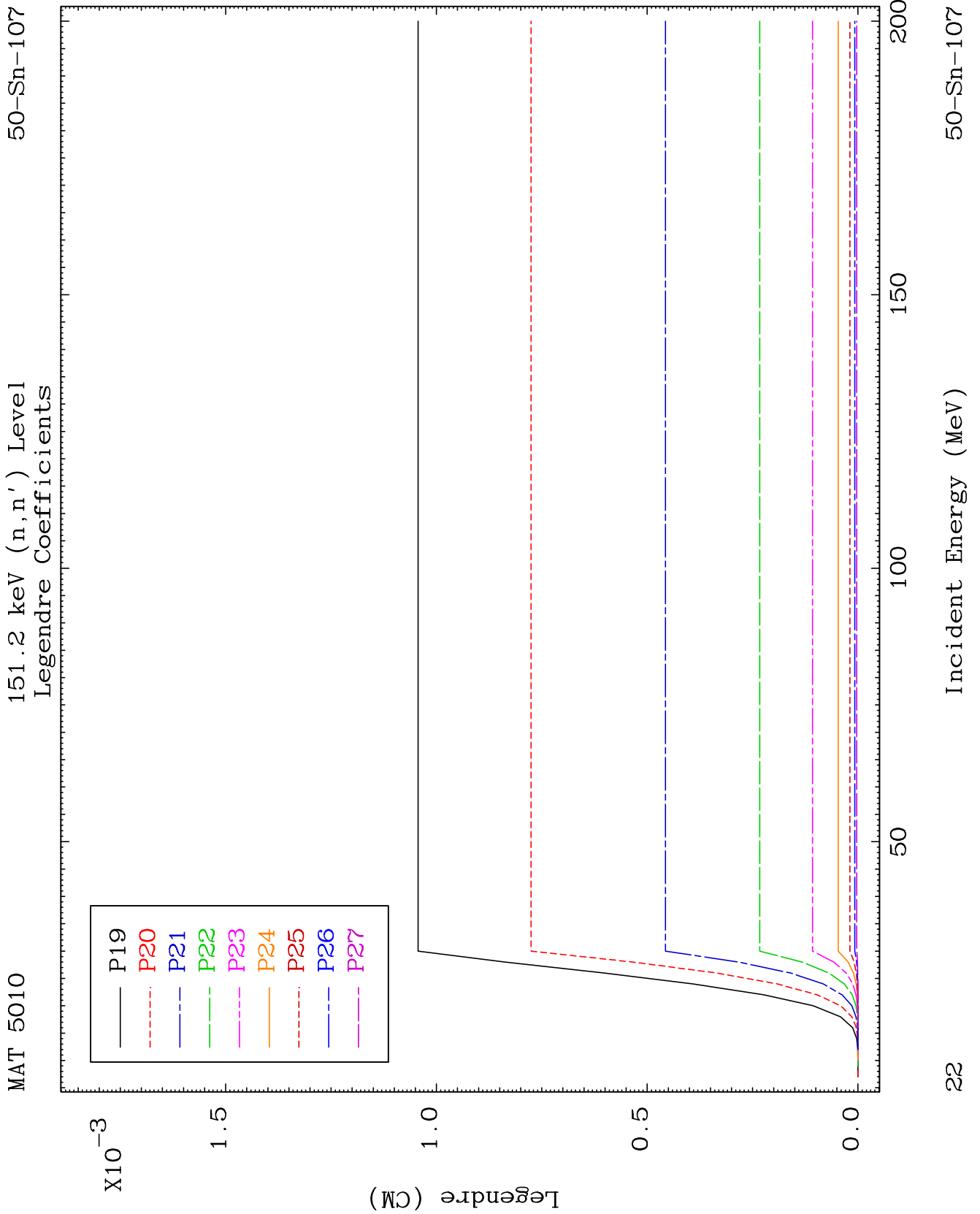
50-Sn-107

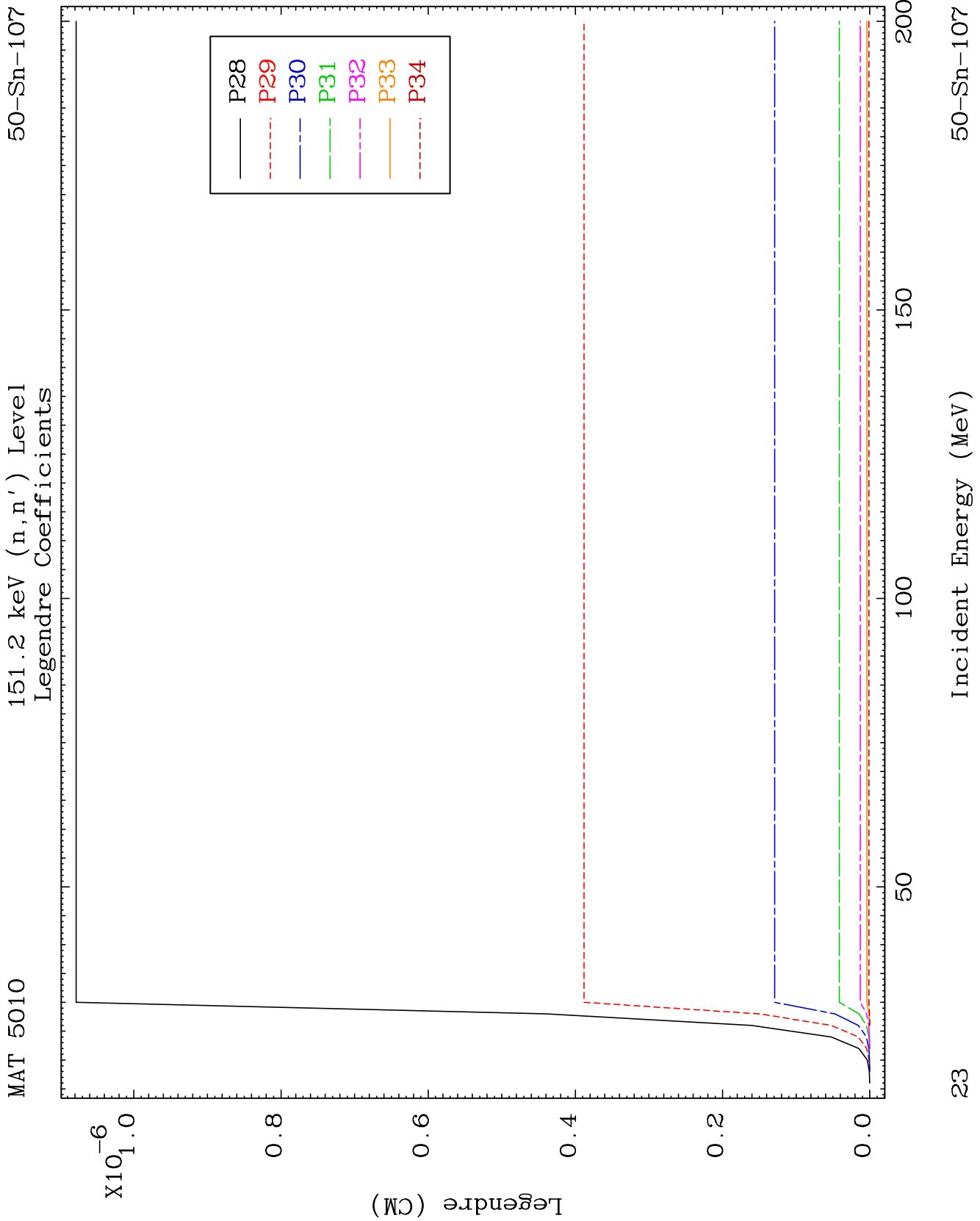


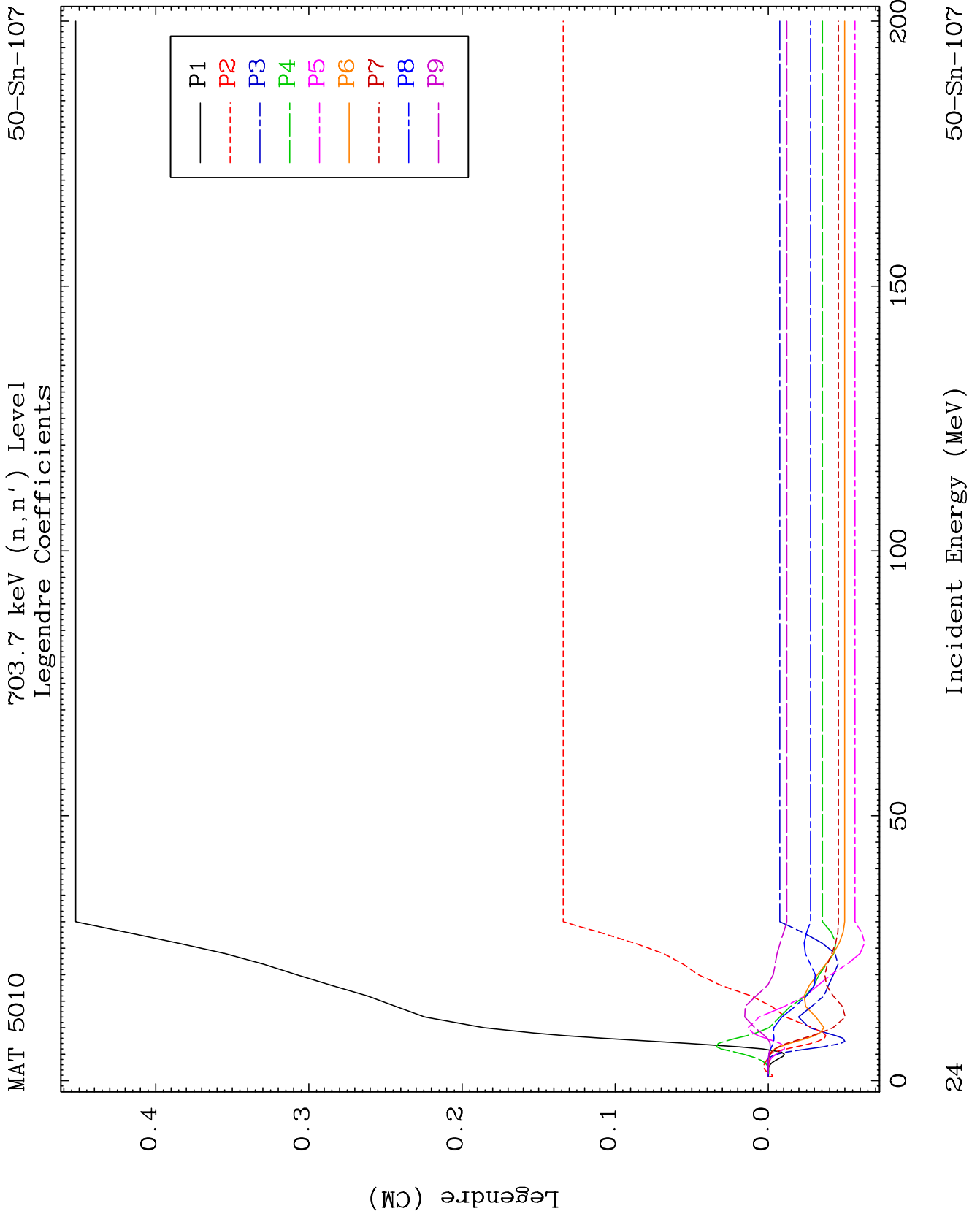
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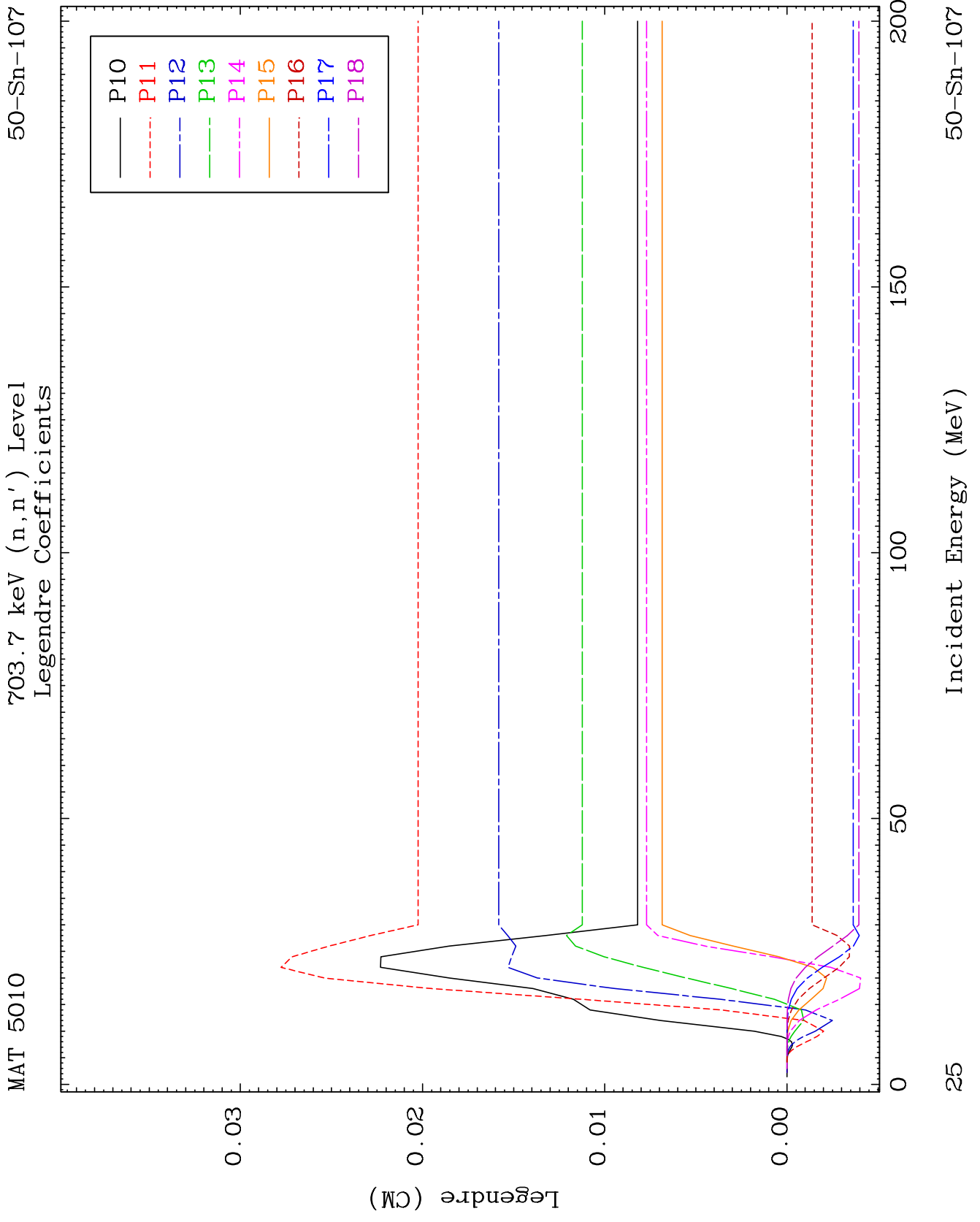
Incident Energy (MeV)

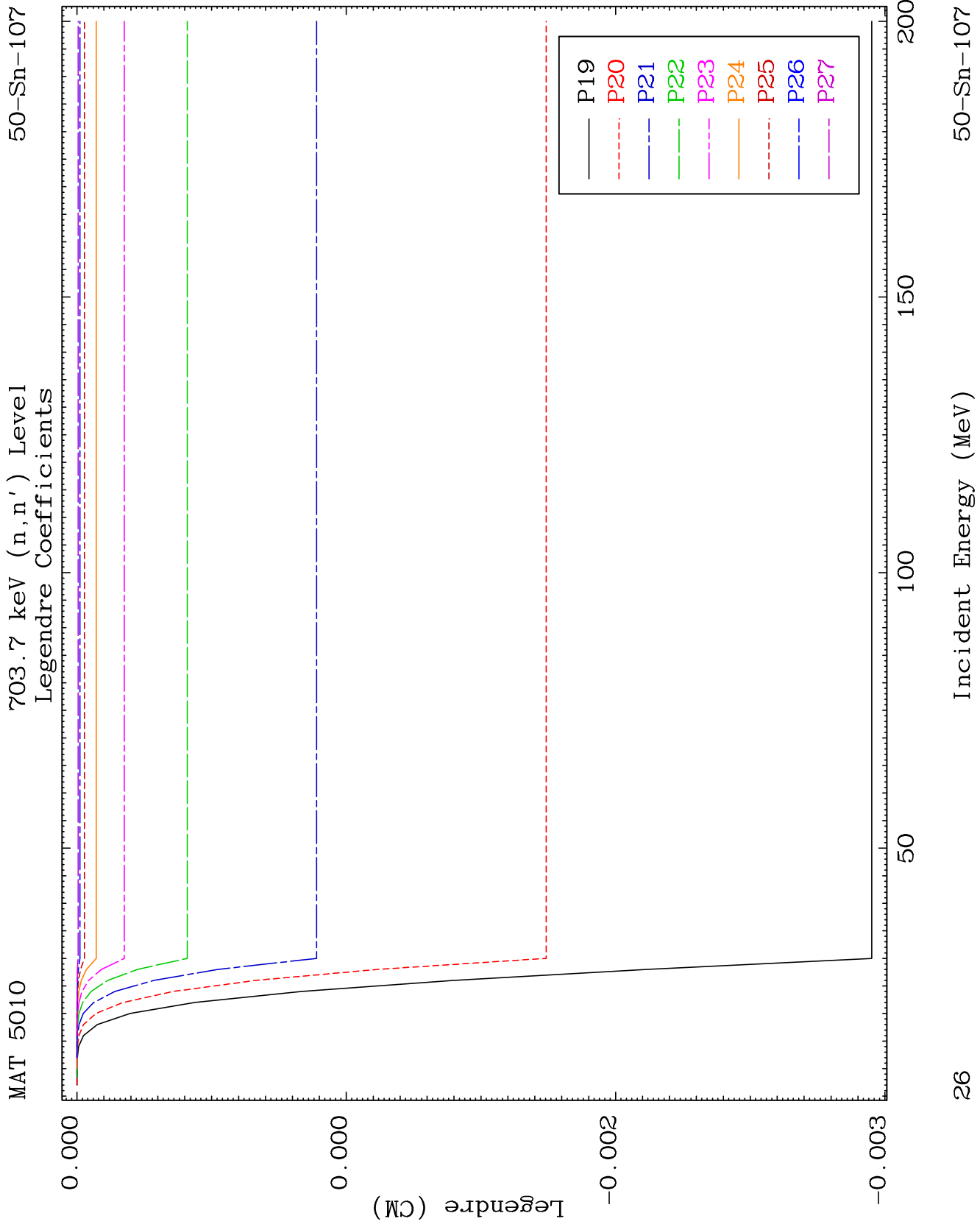
50-Sn-107

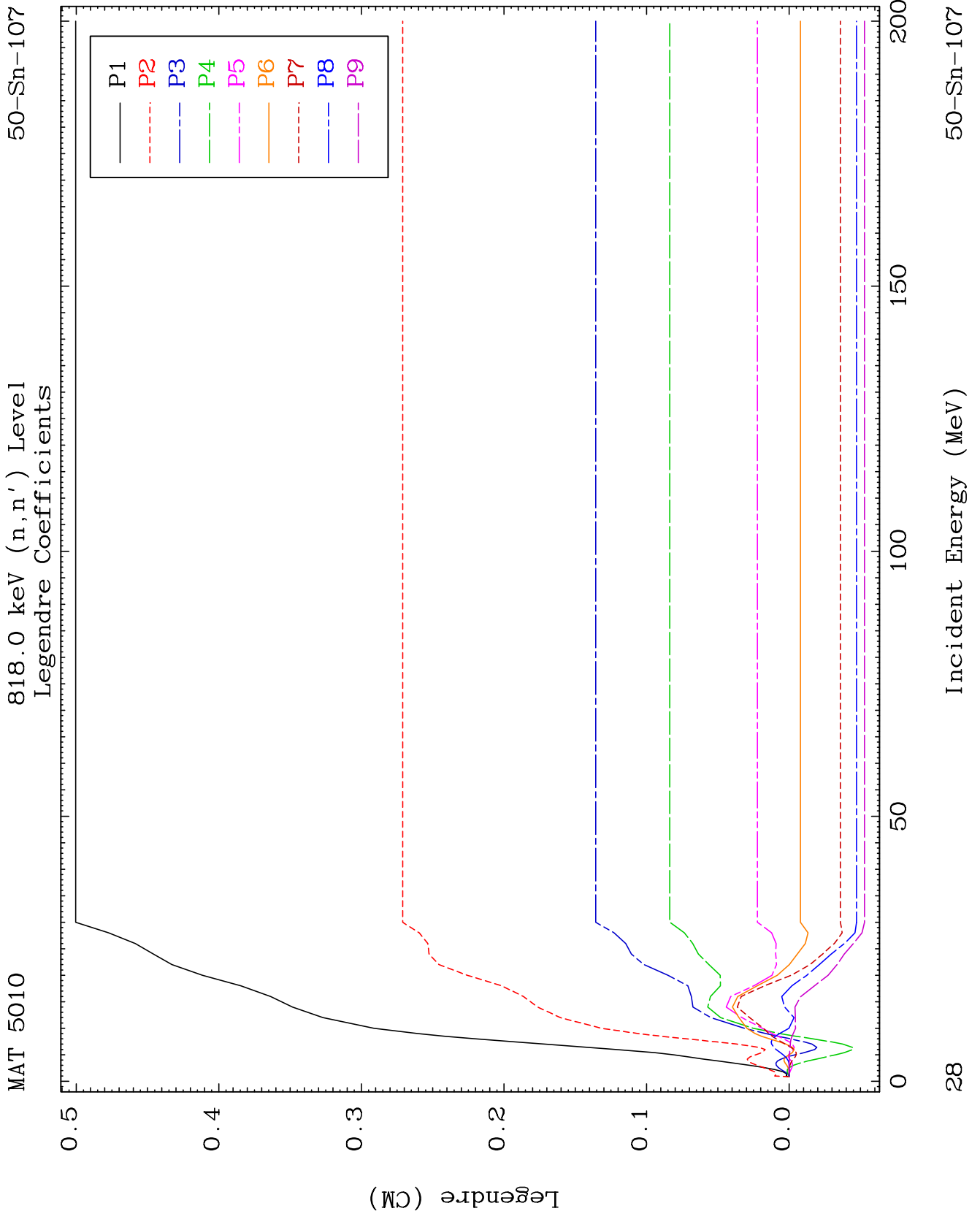


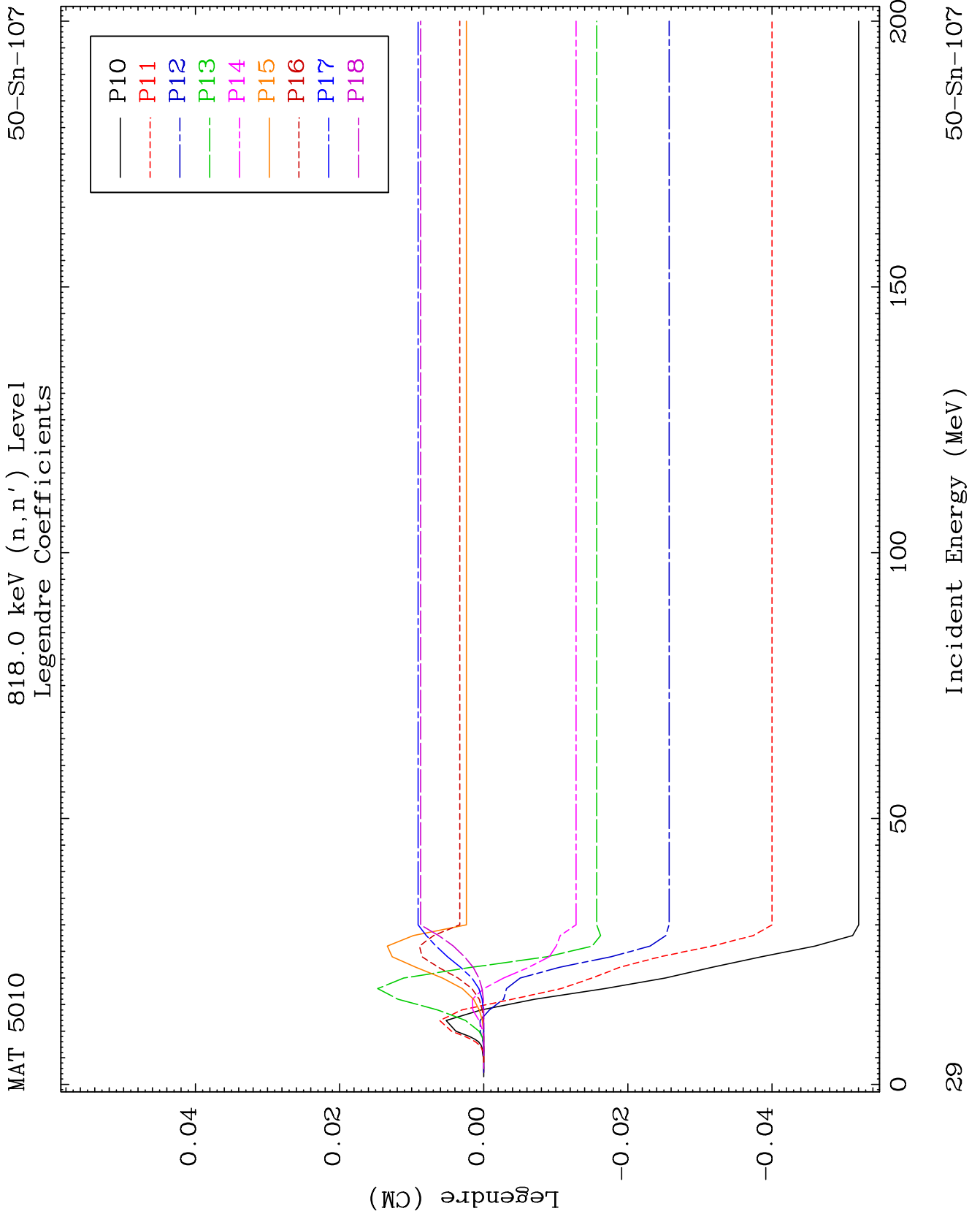


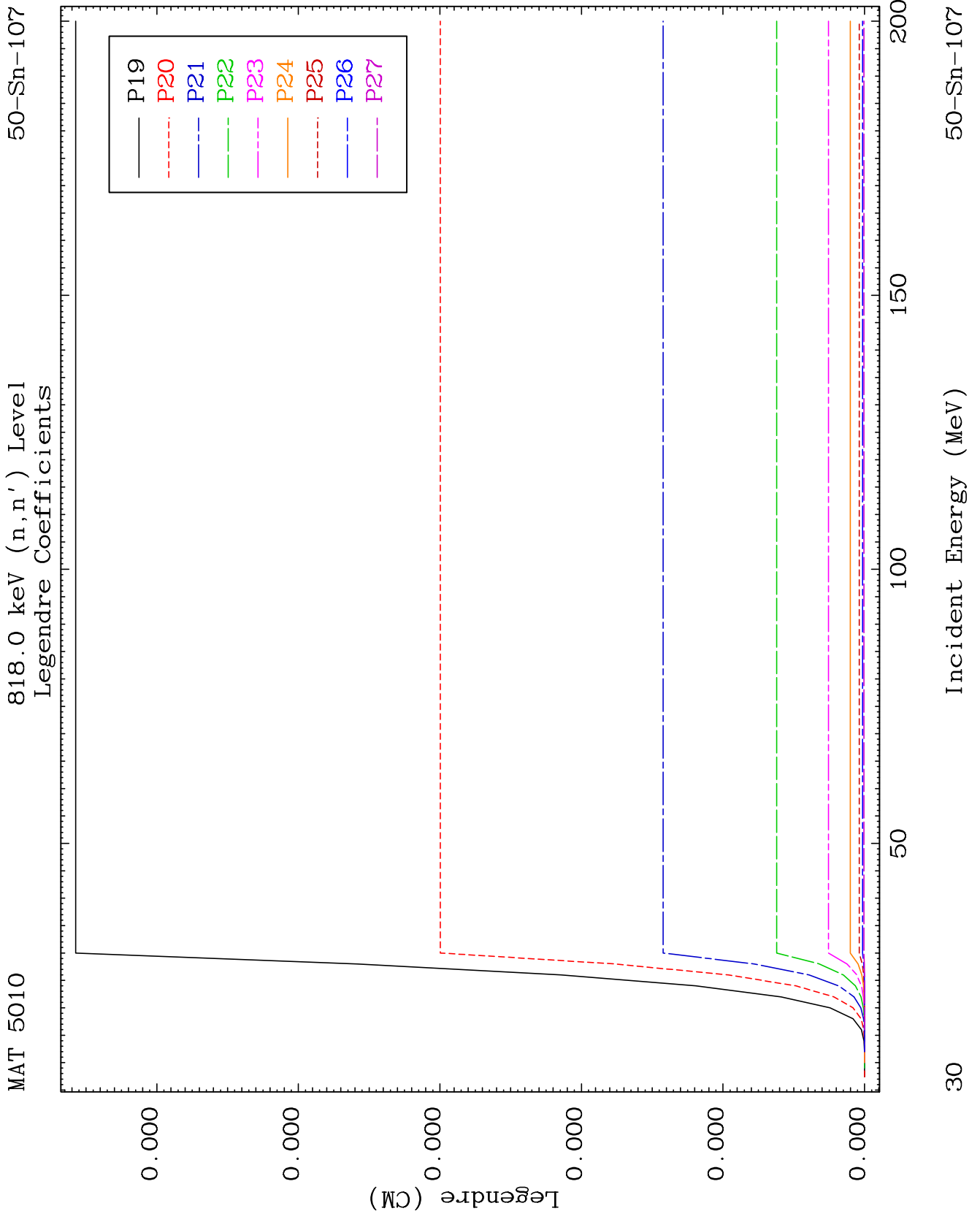


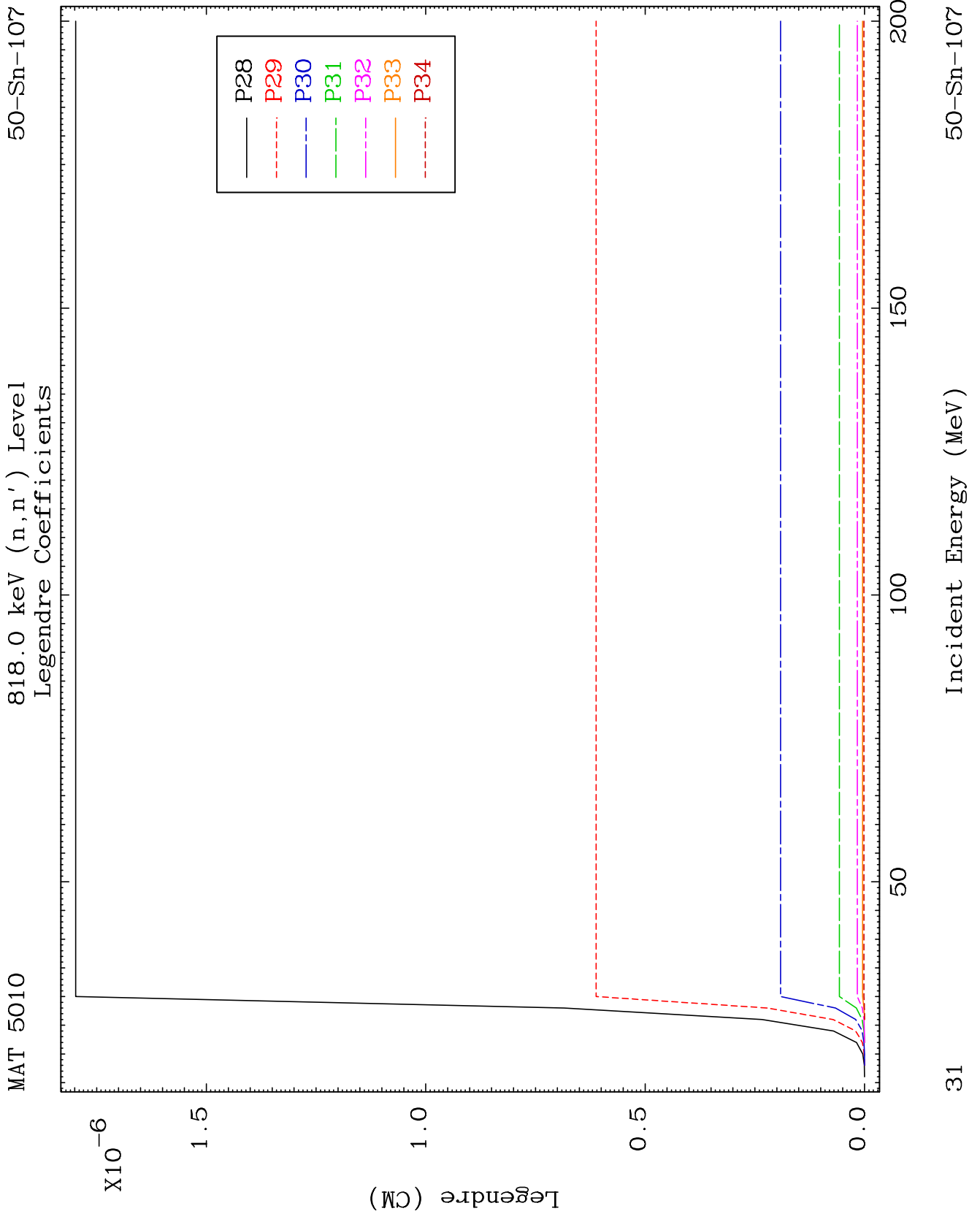


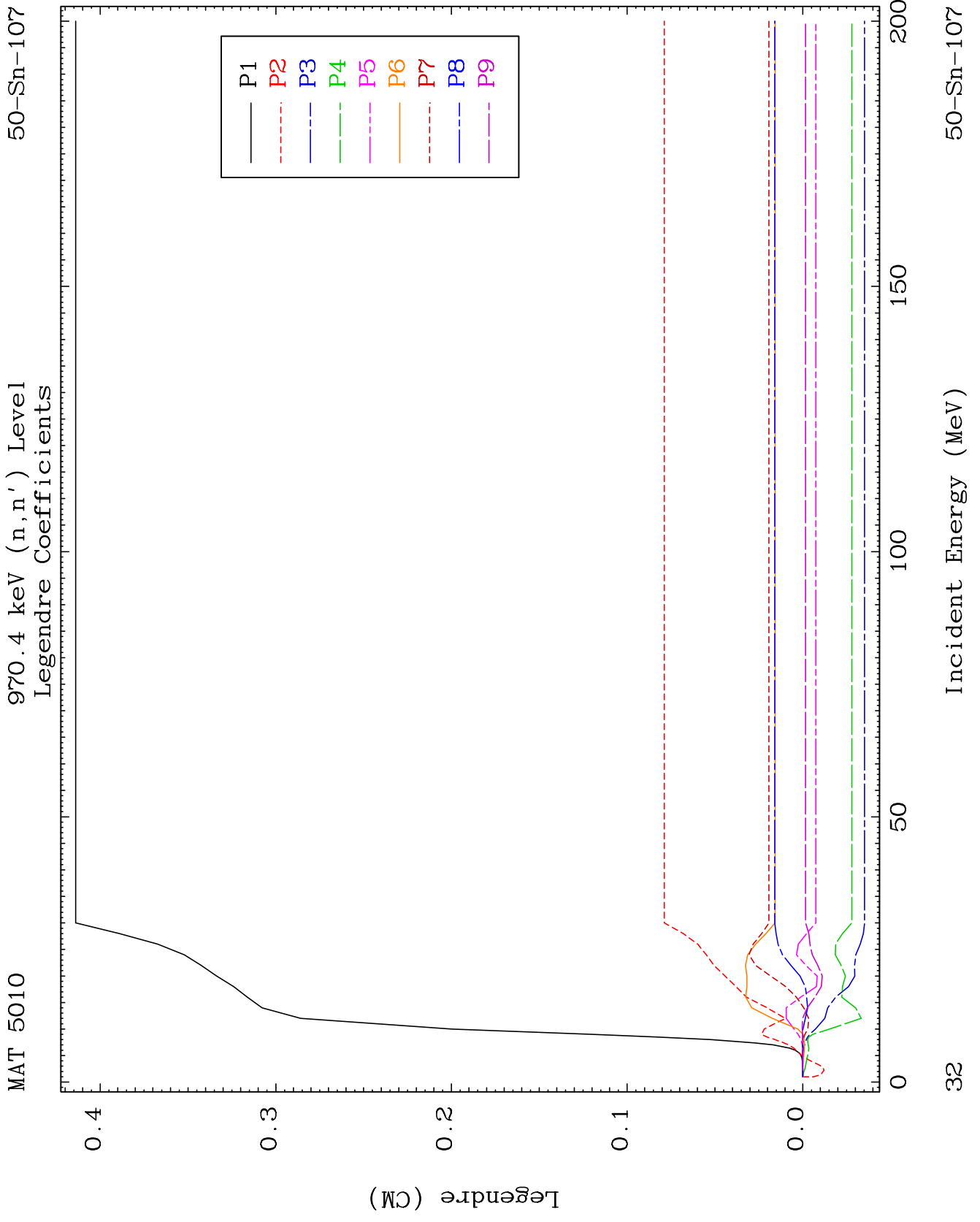


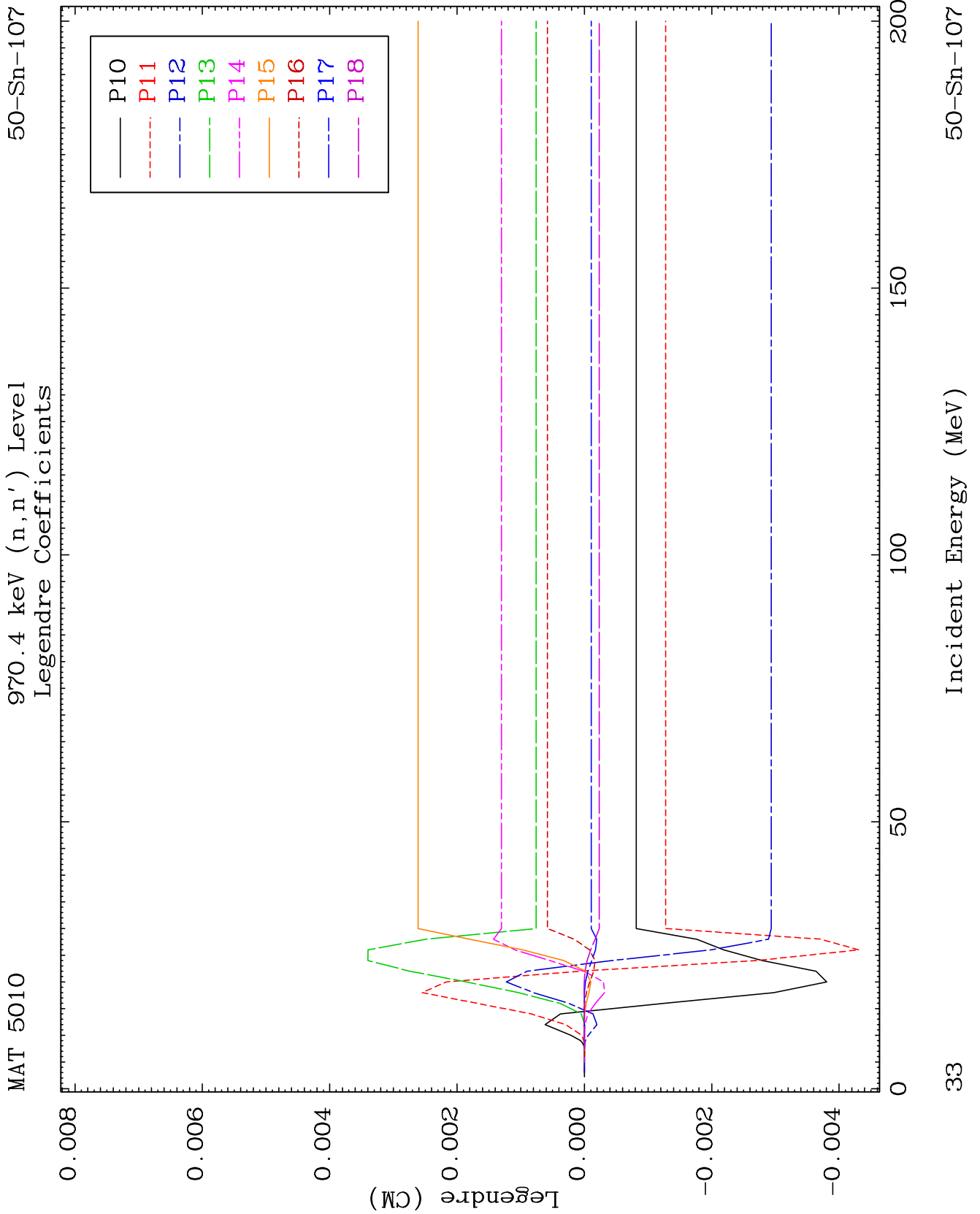


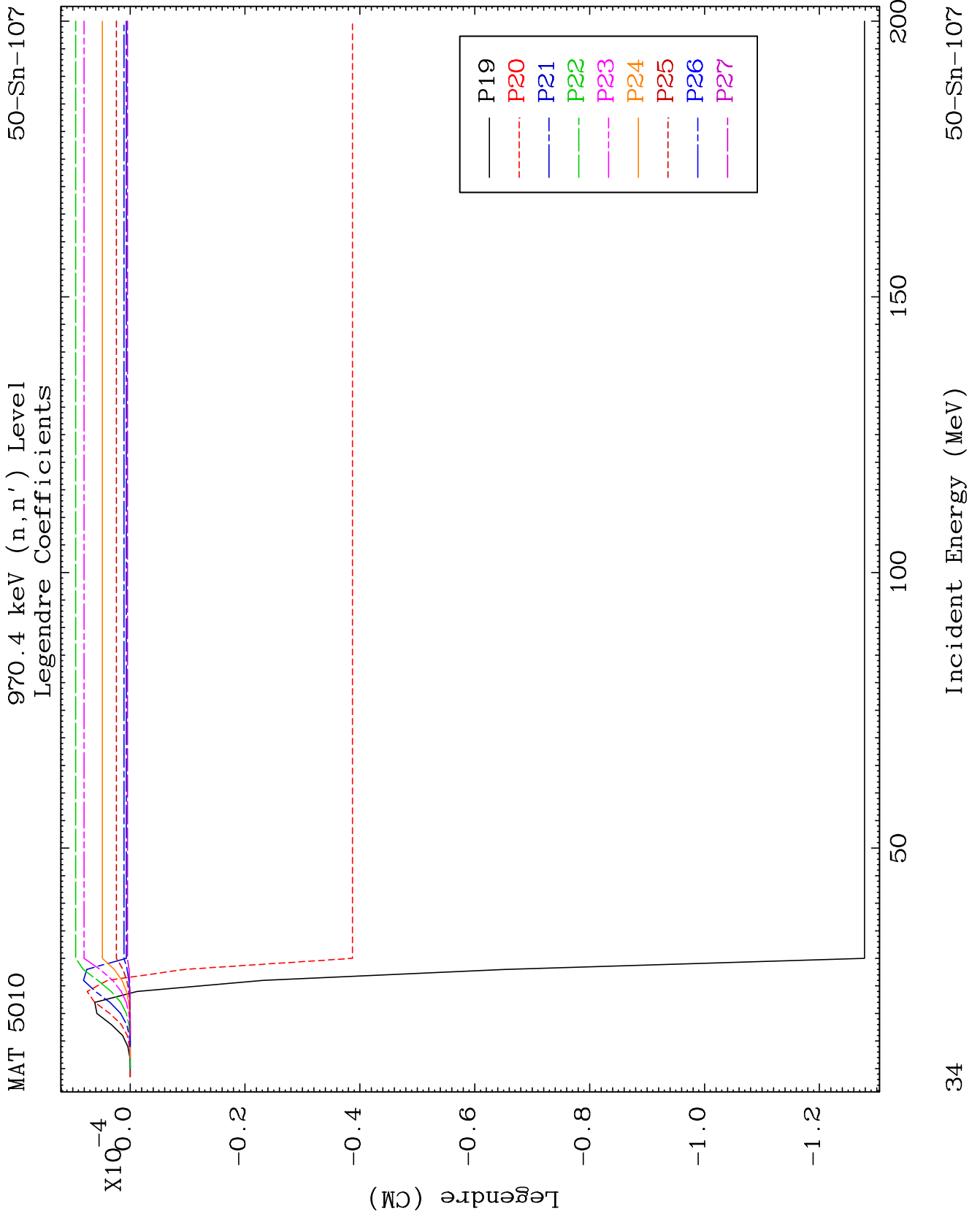


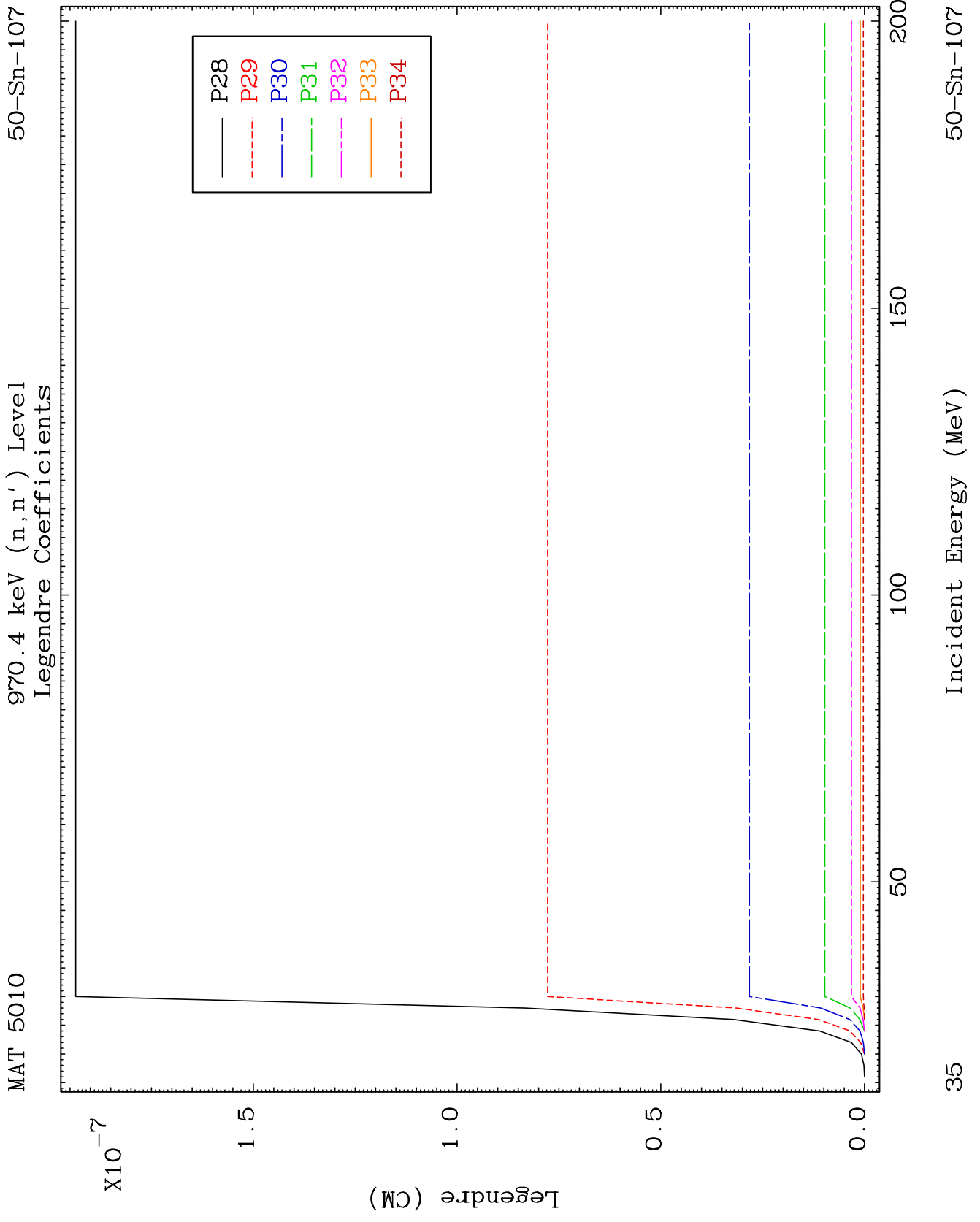


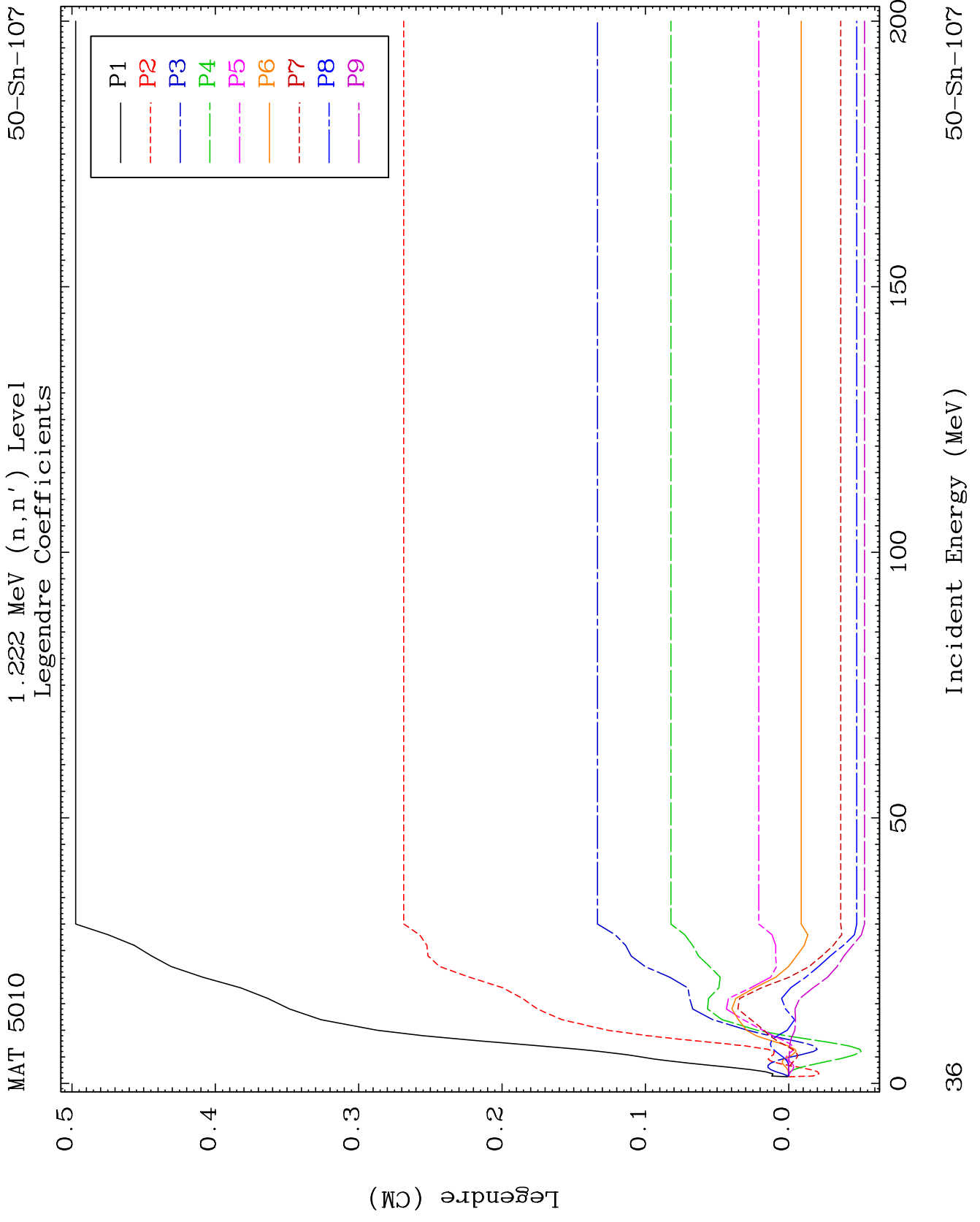


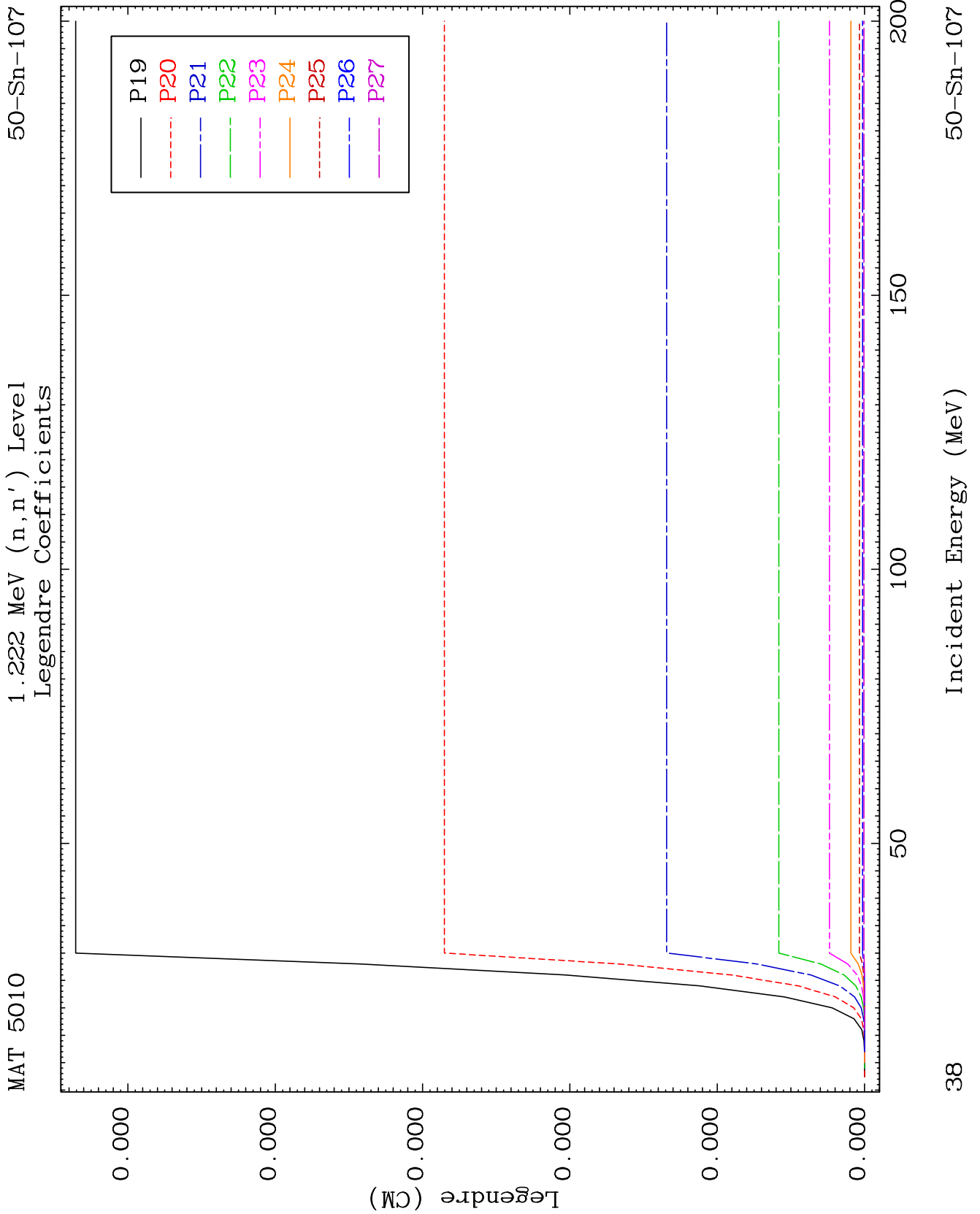


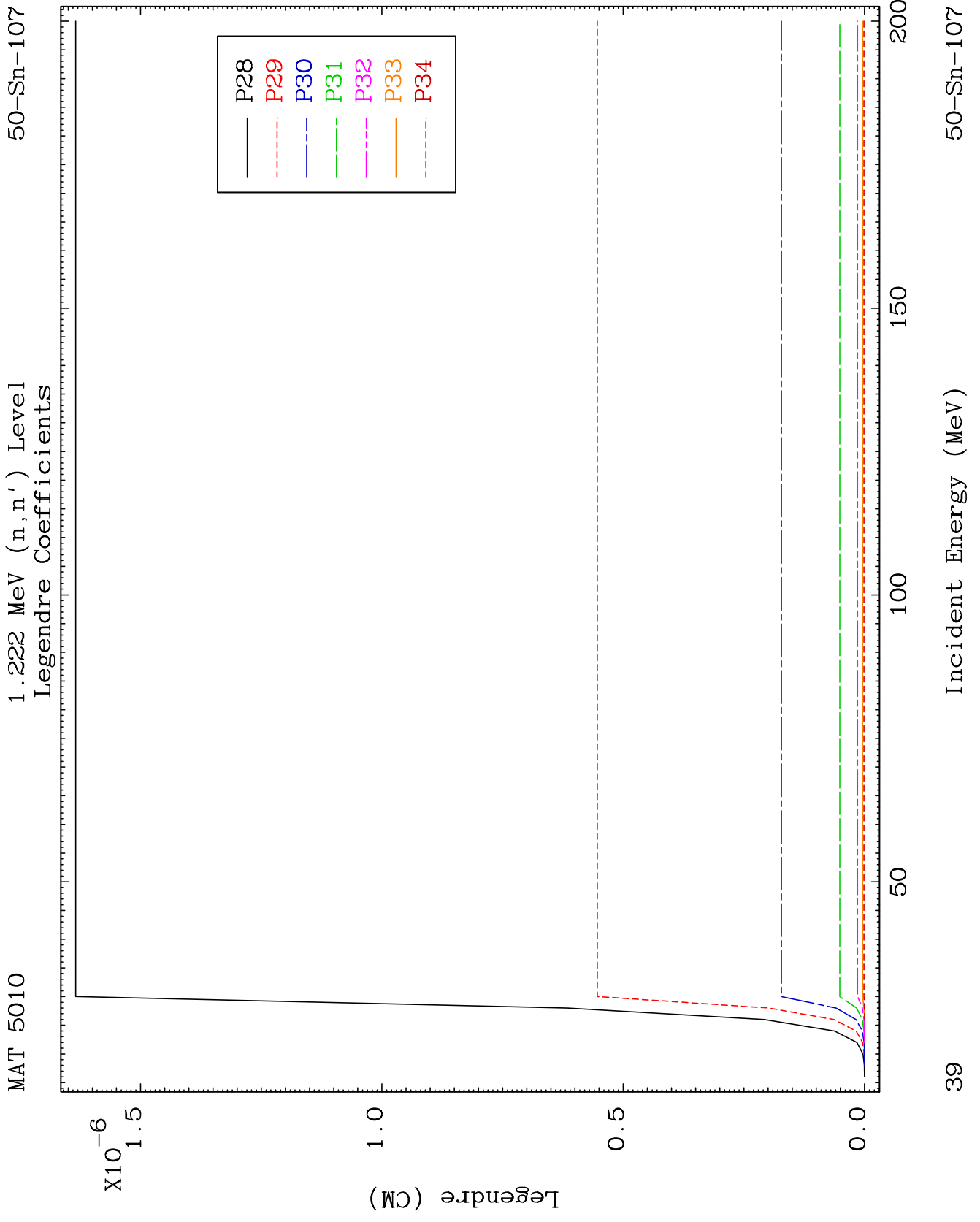


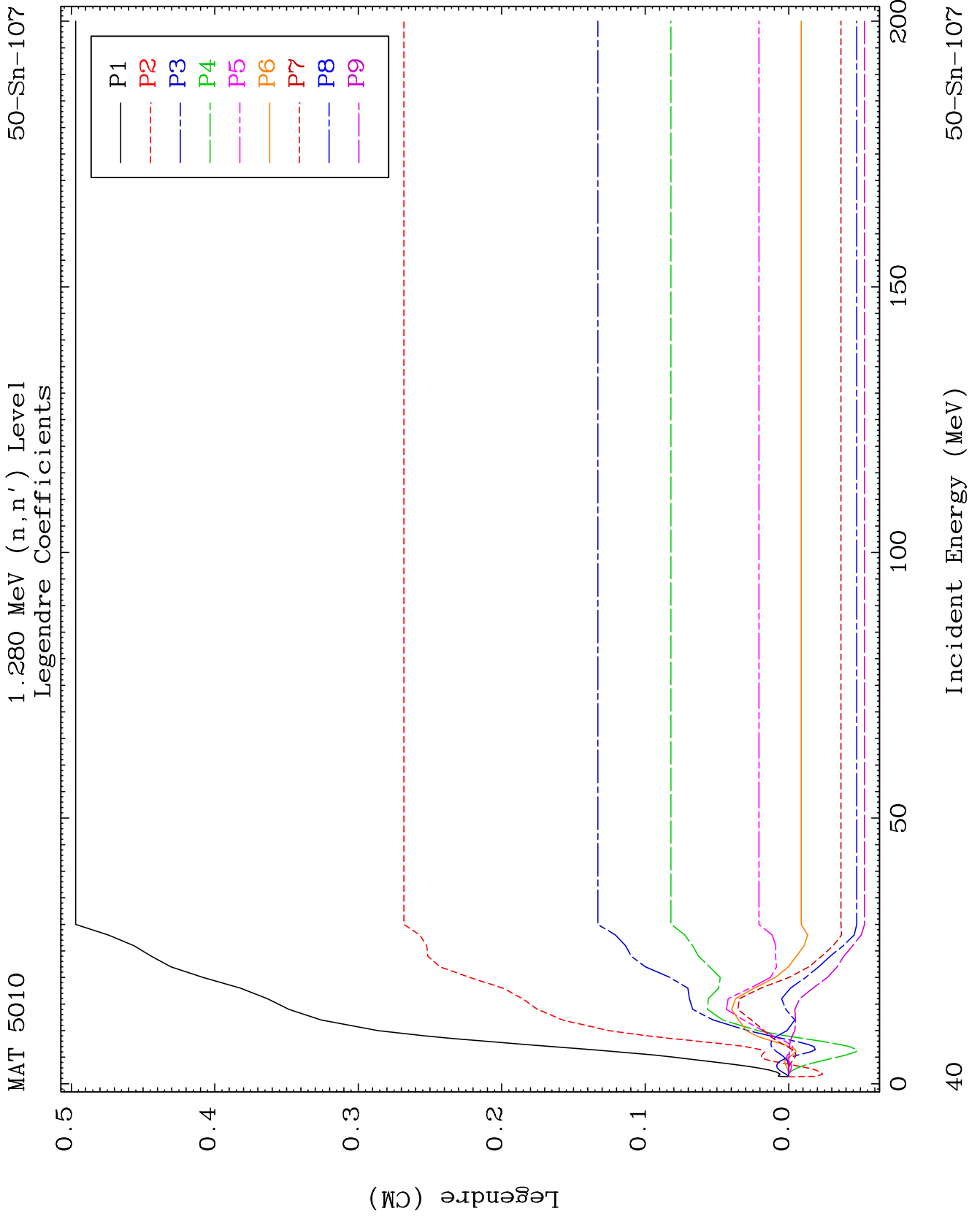


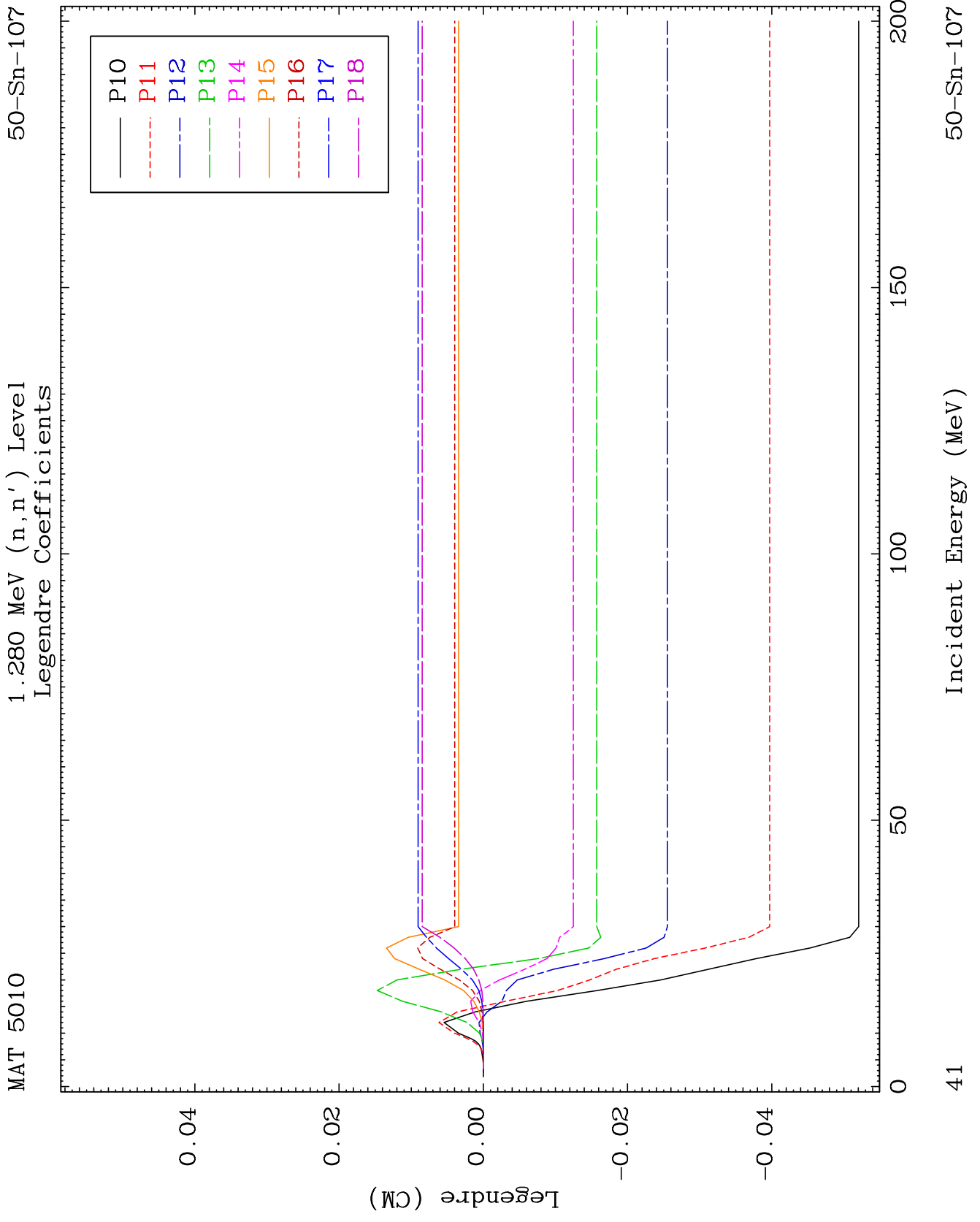


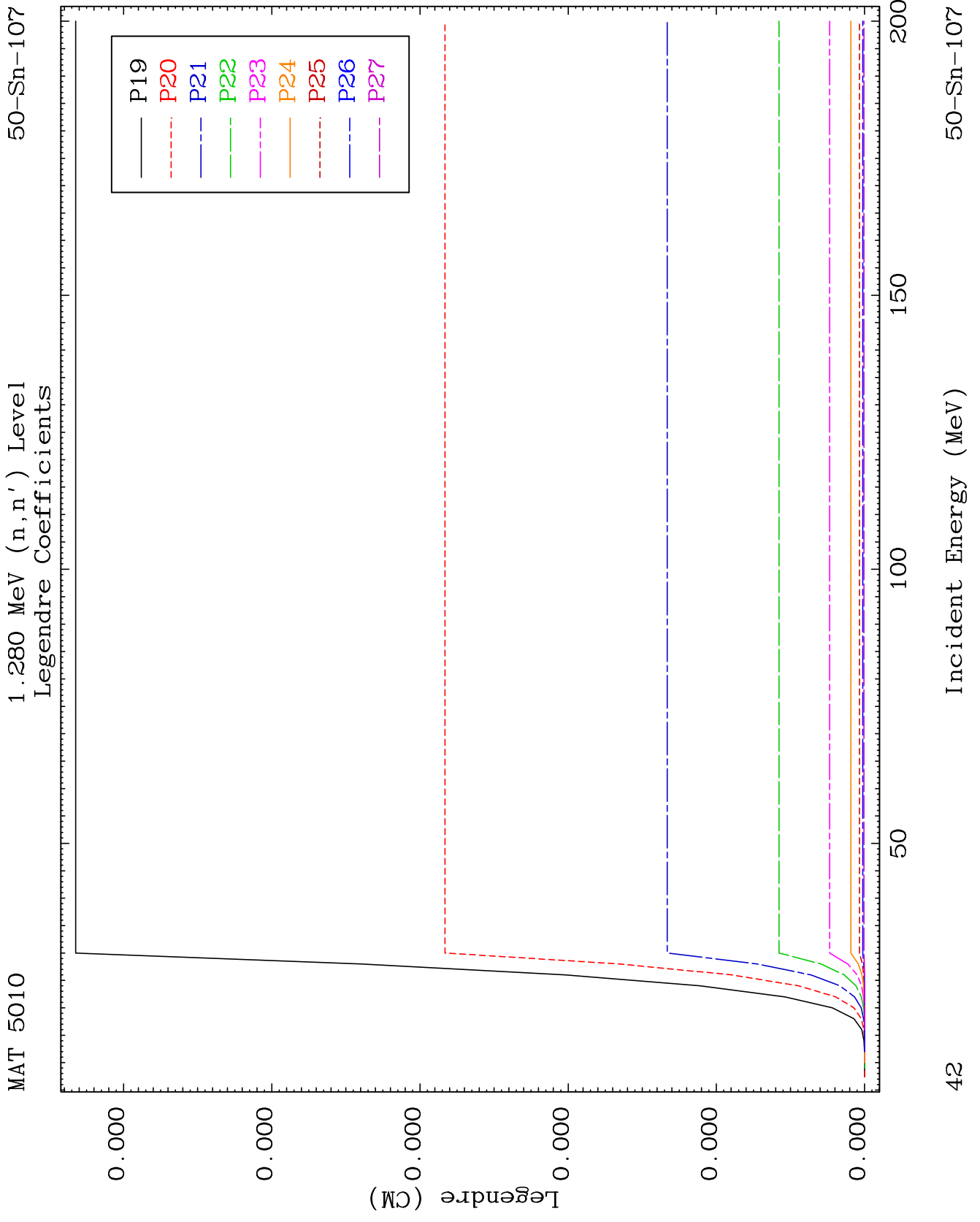


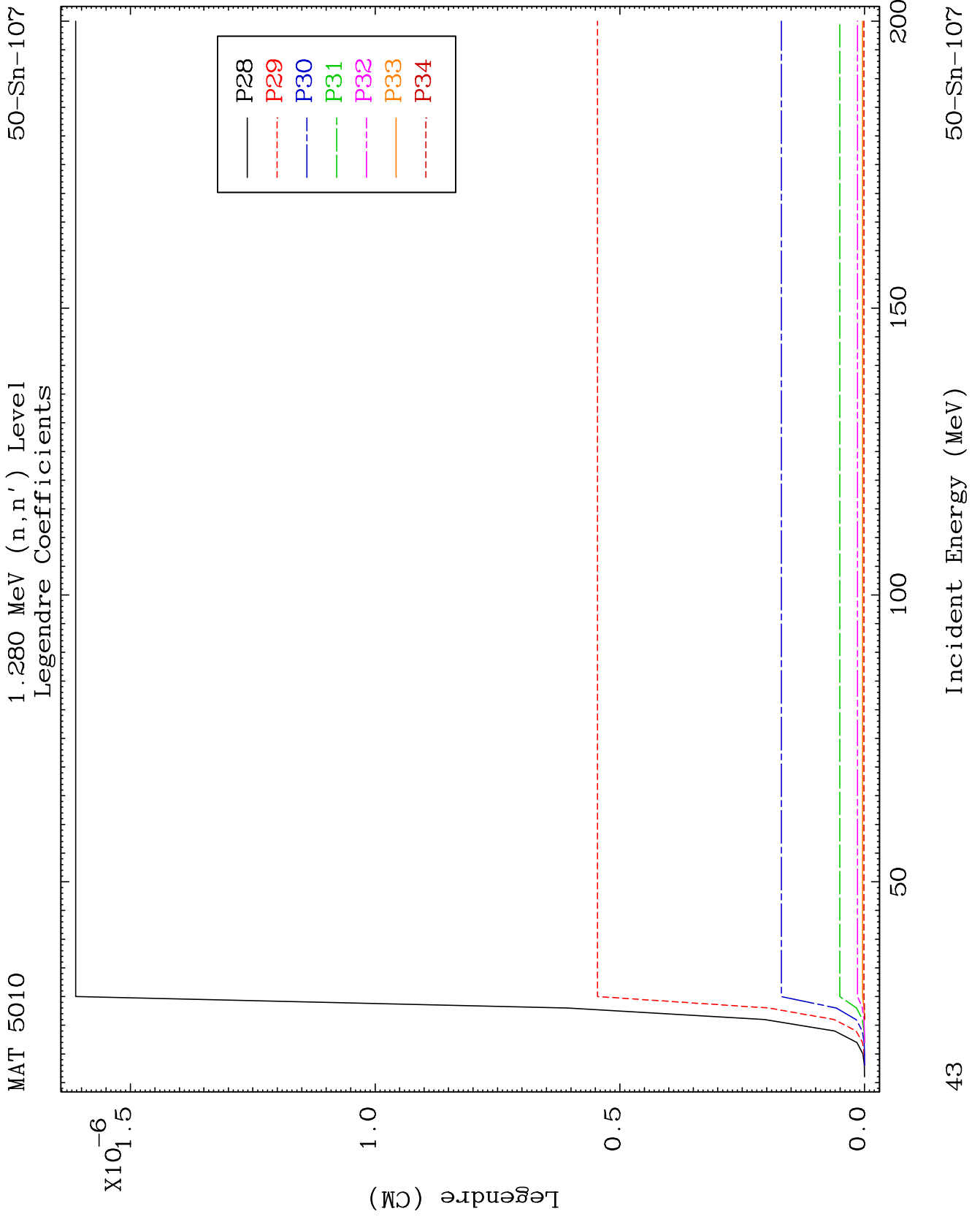


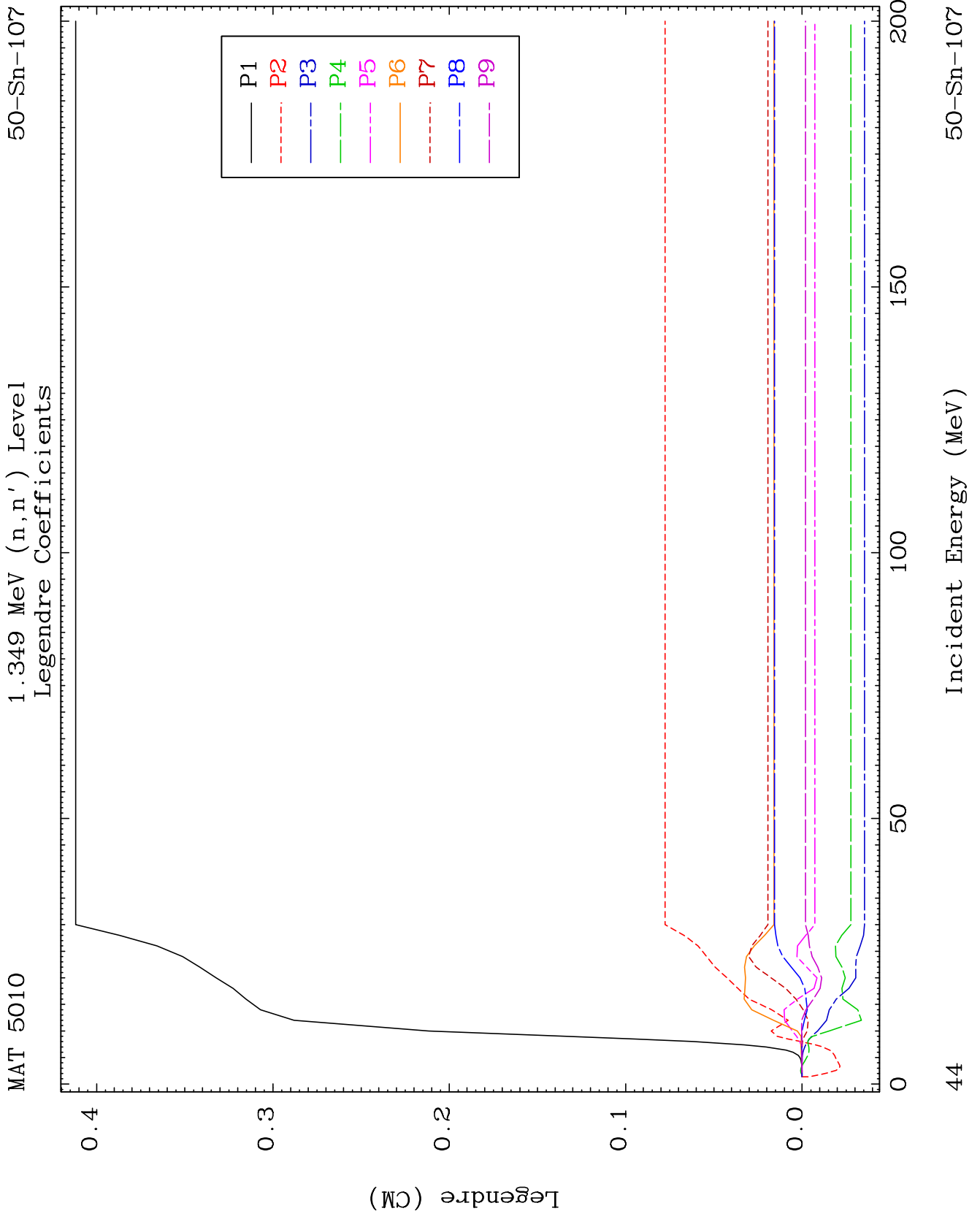


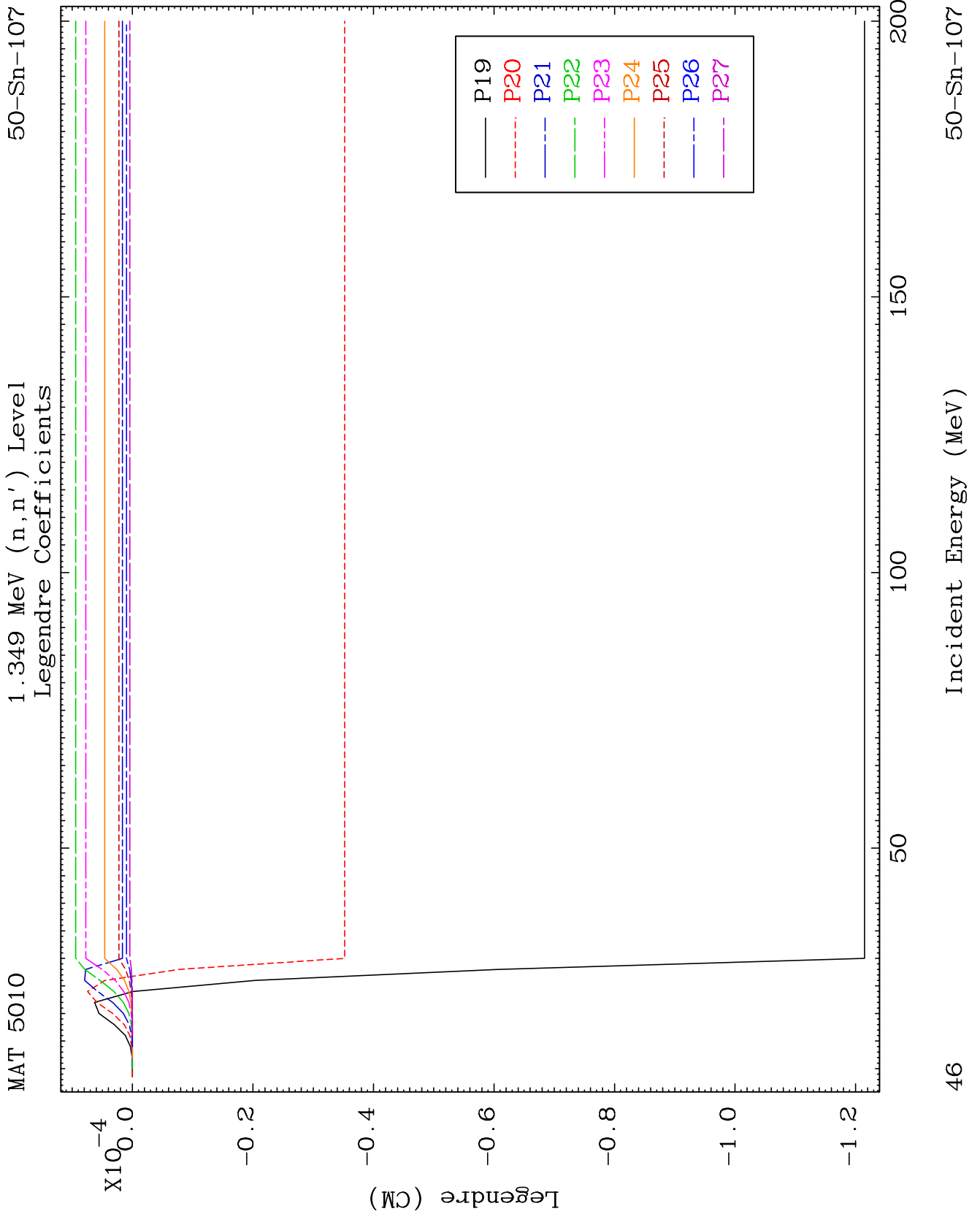


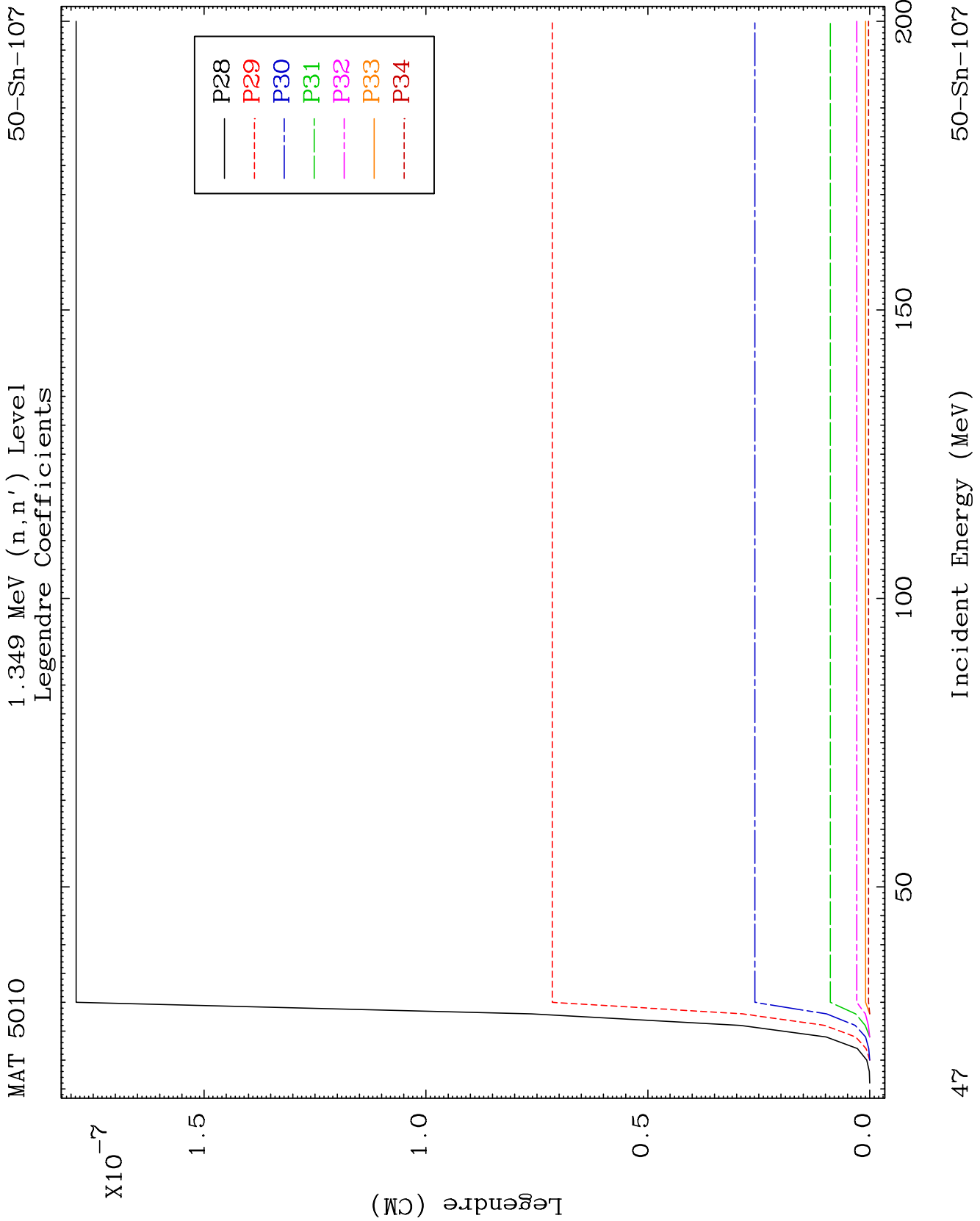


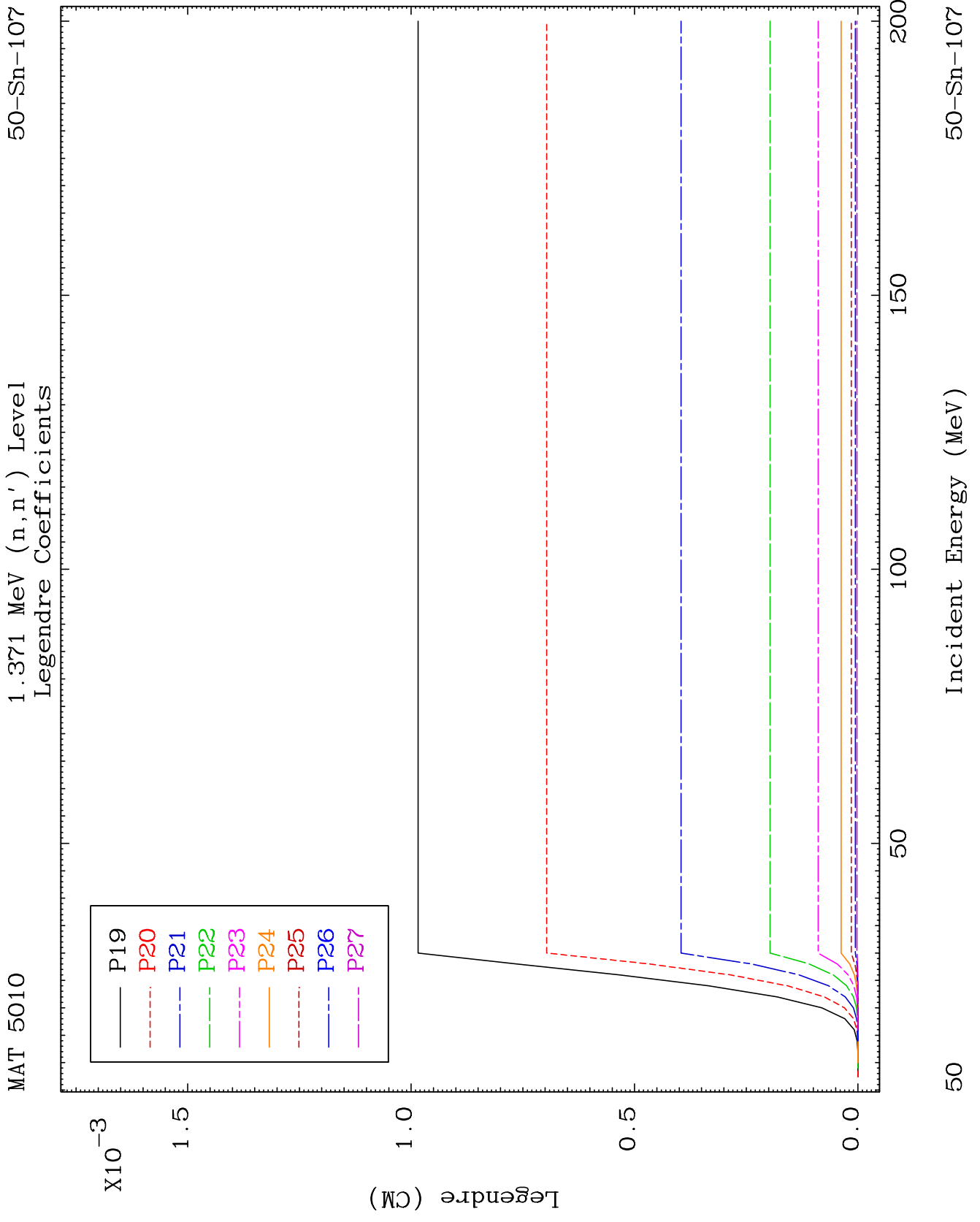


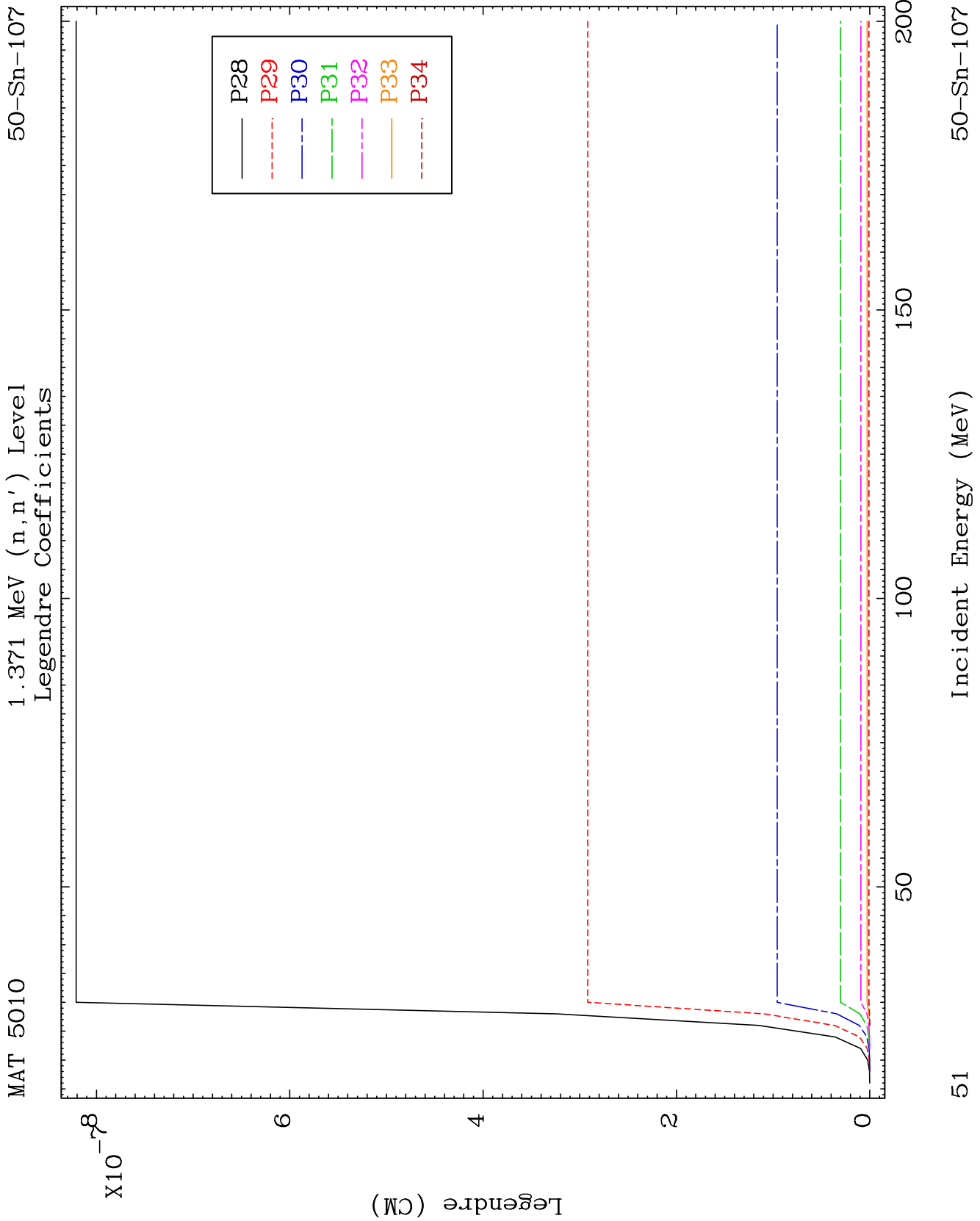


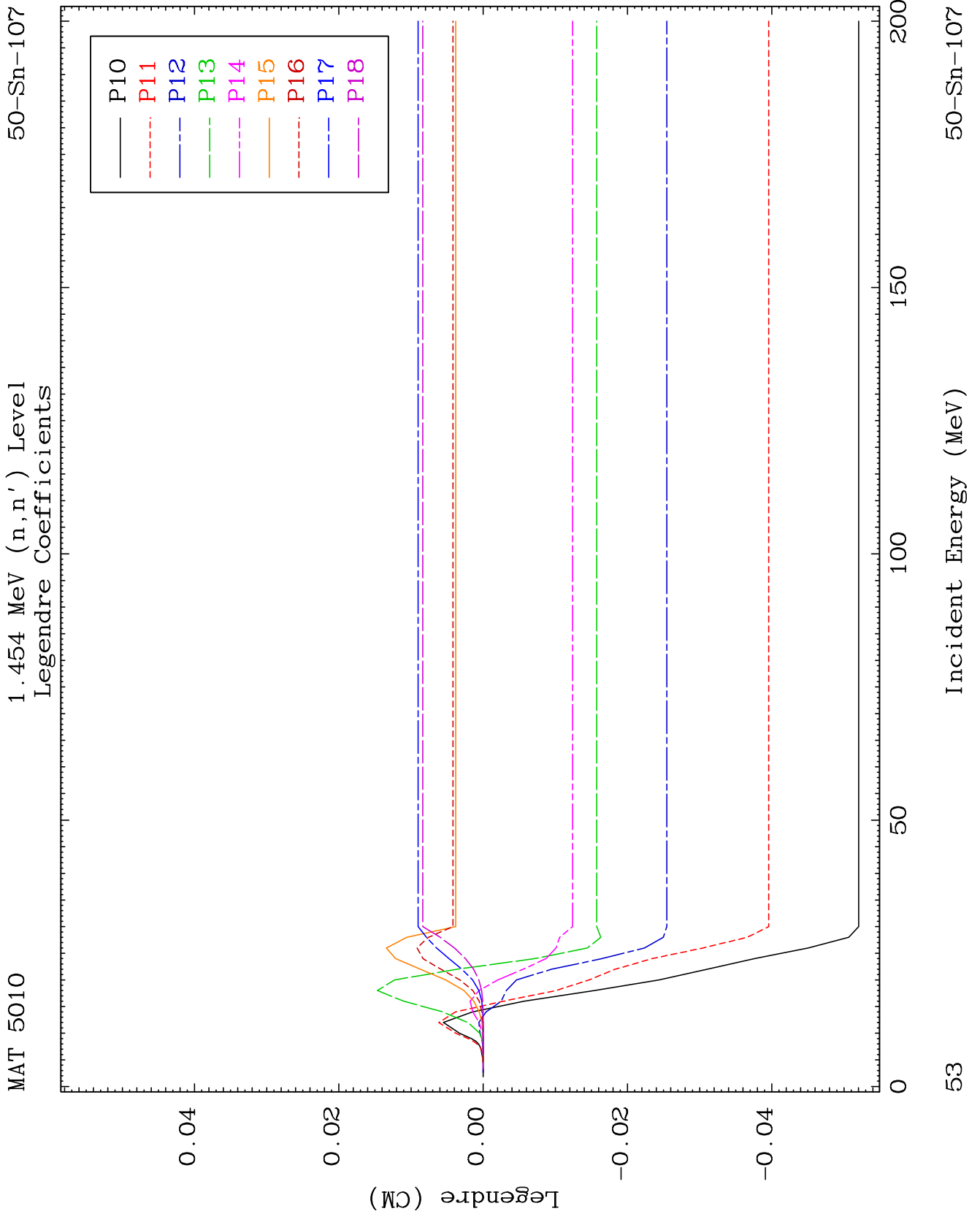


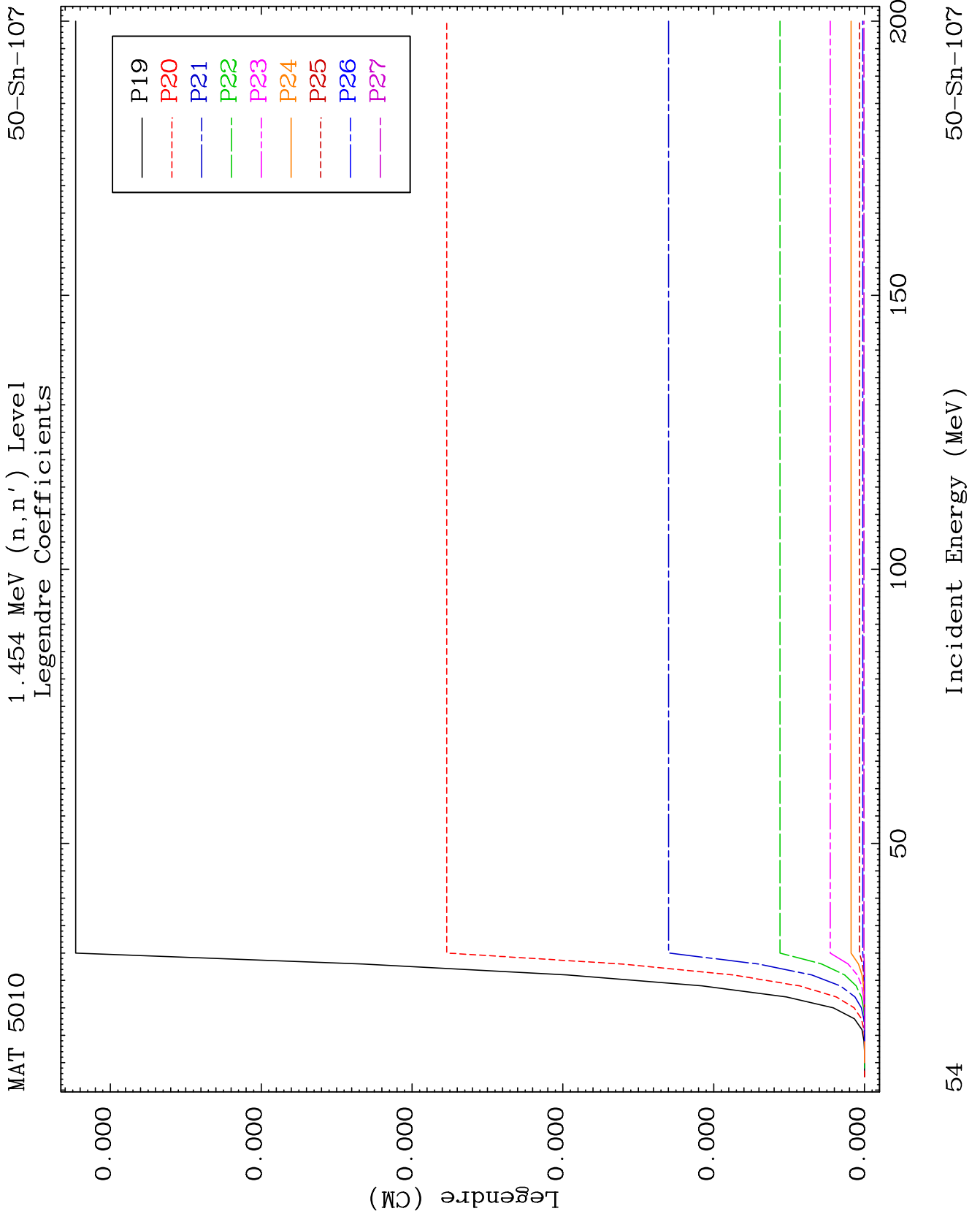


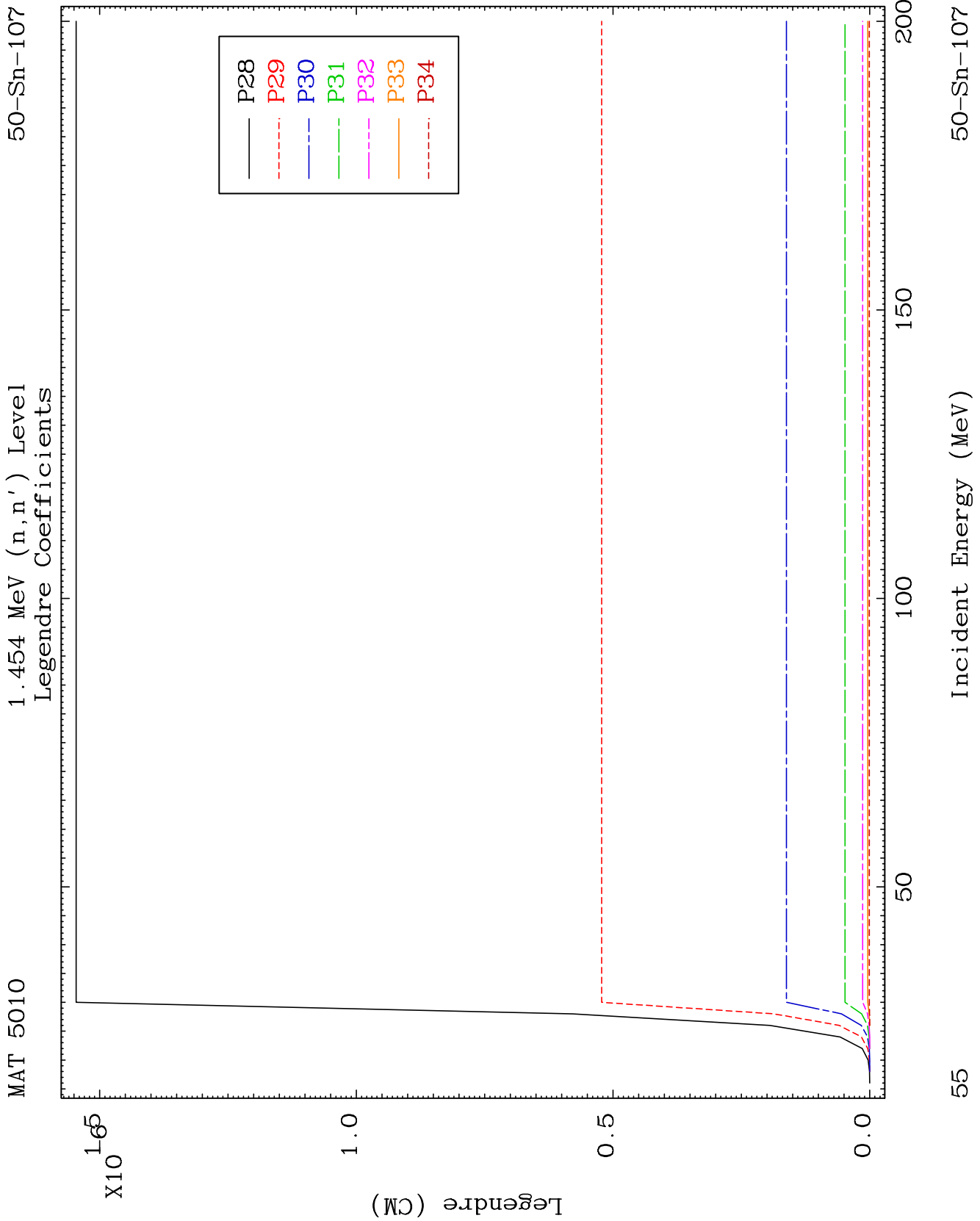


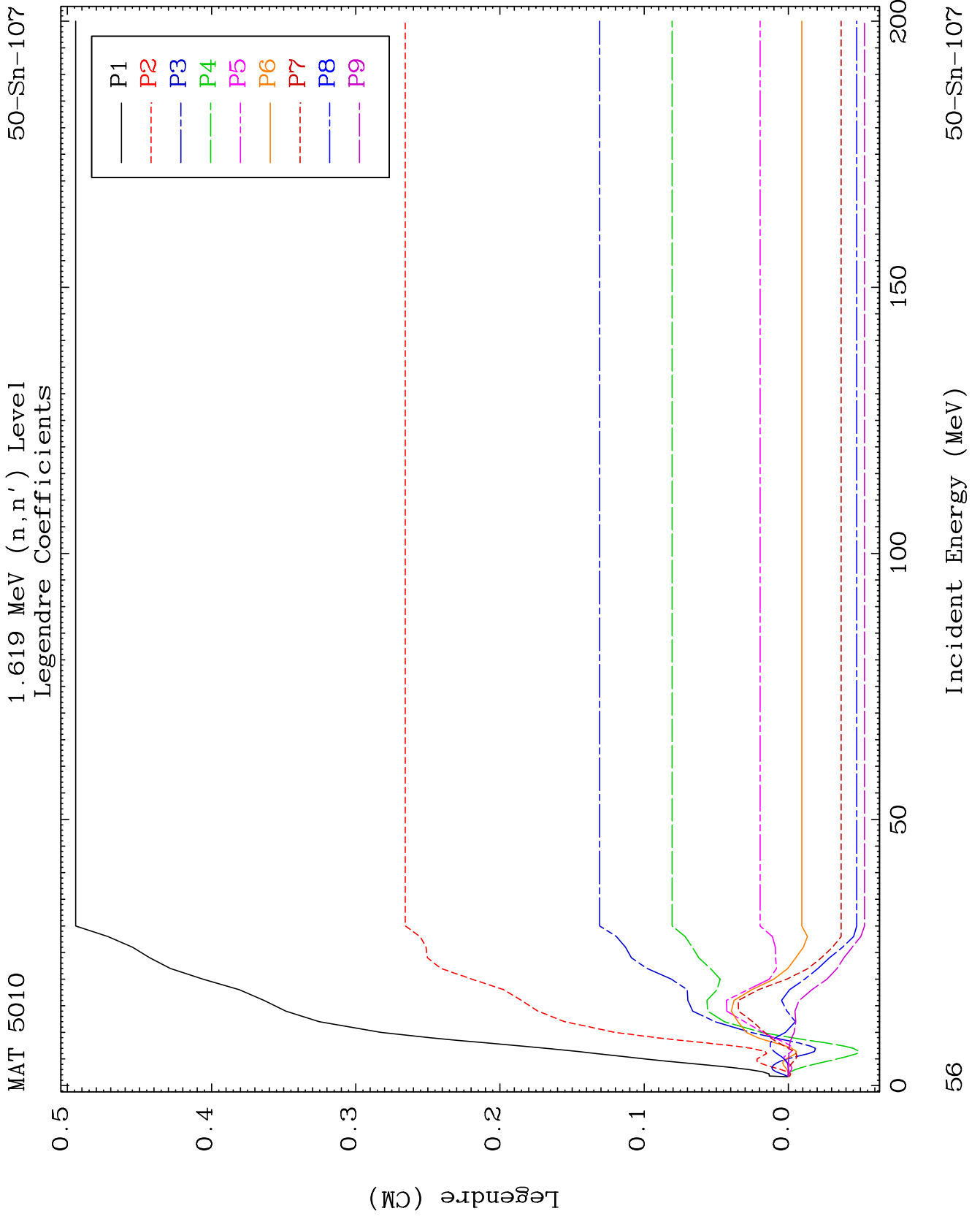


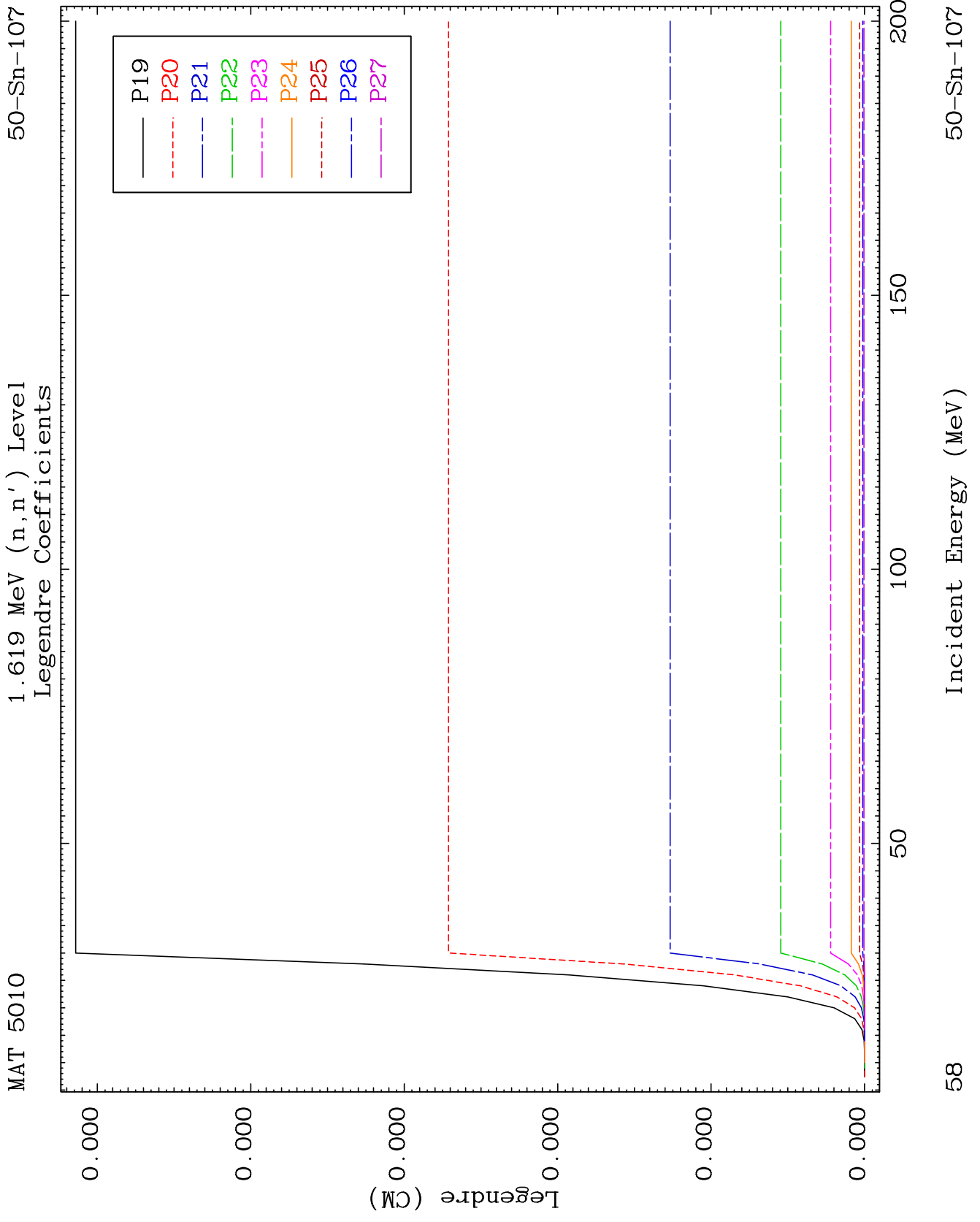


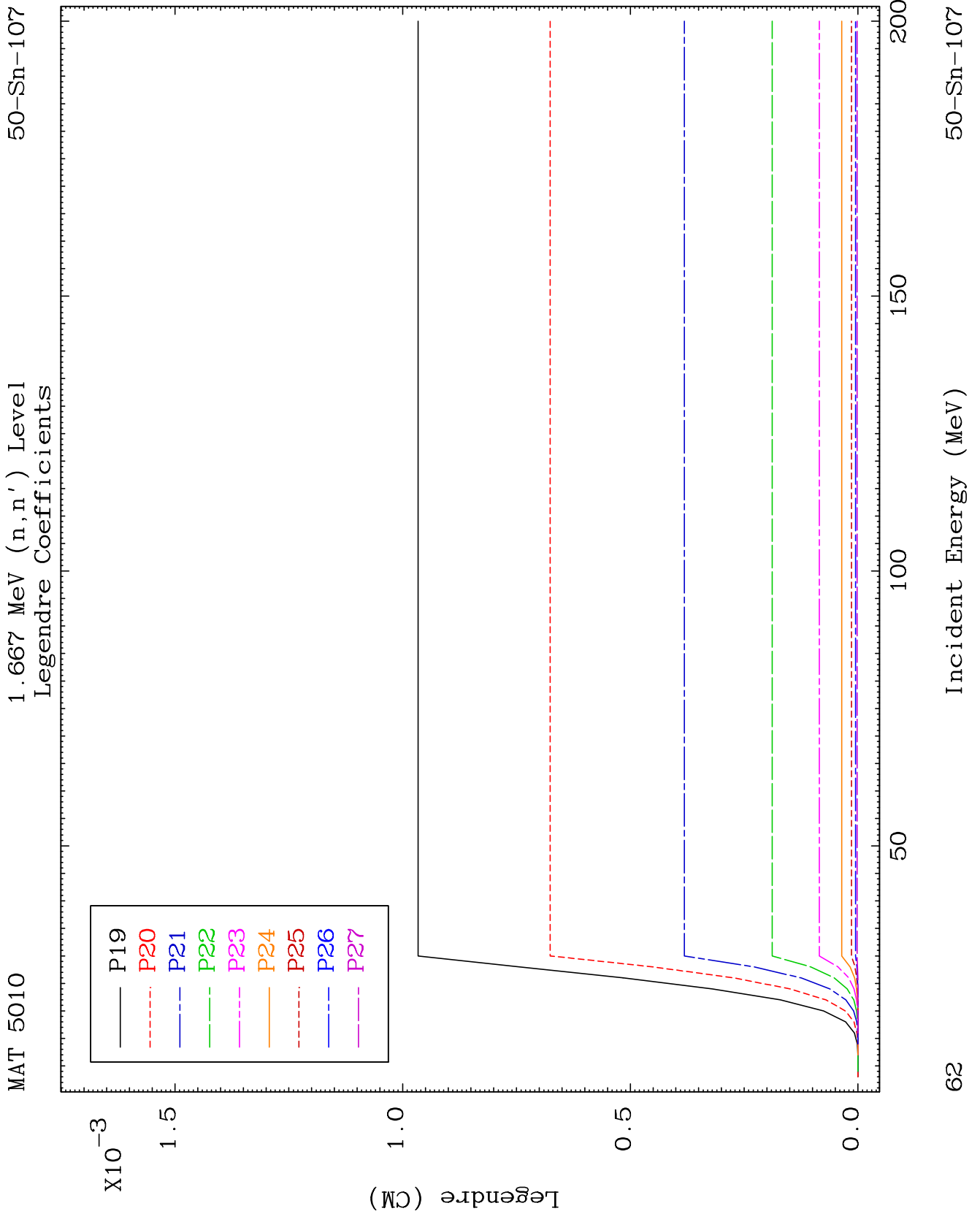


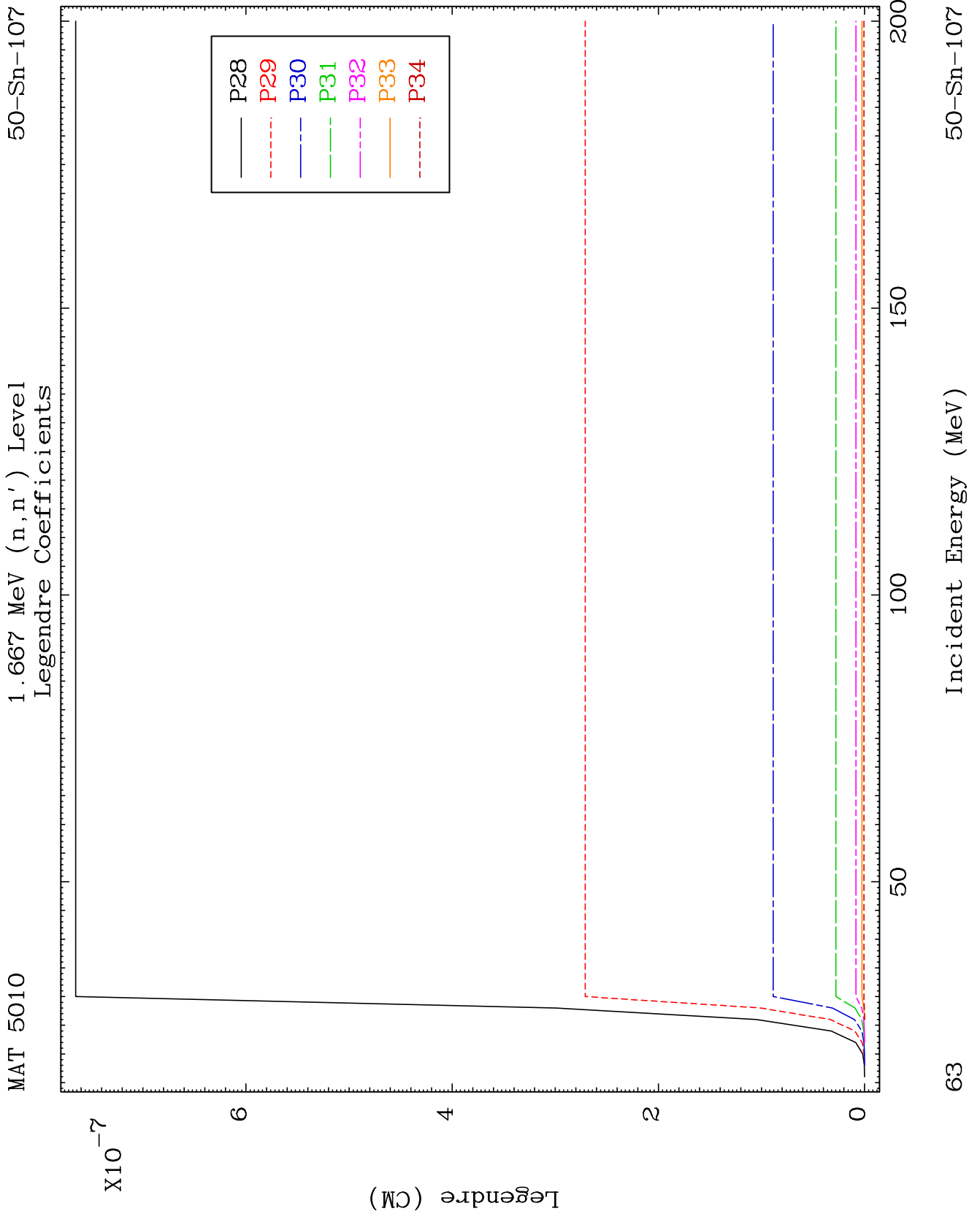


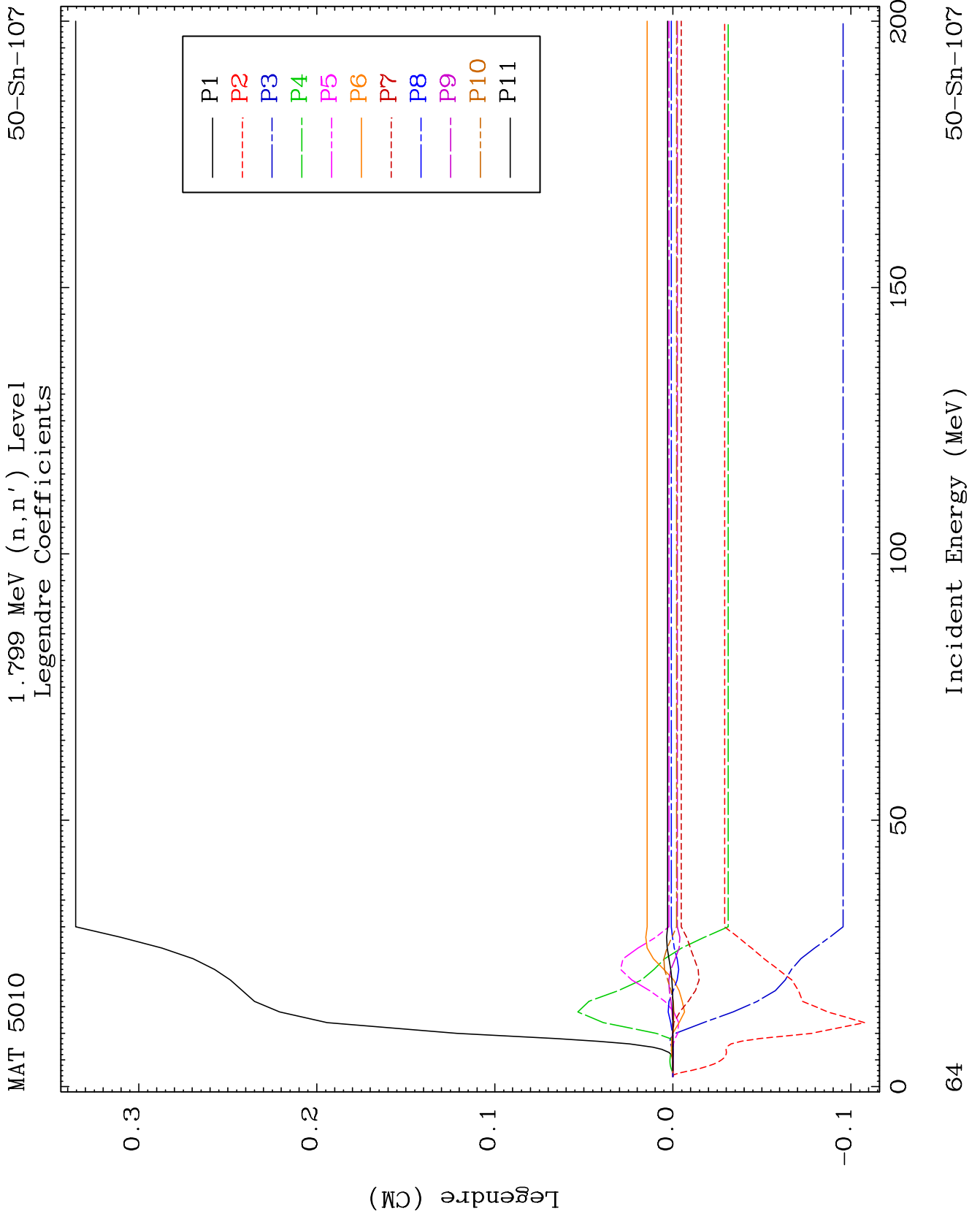








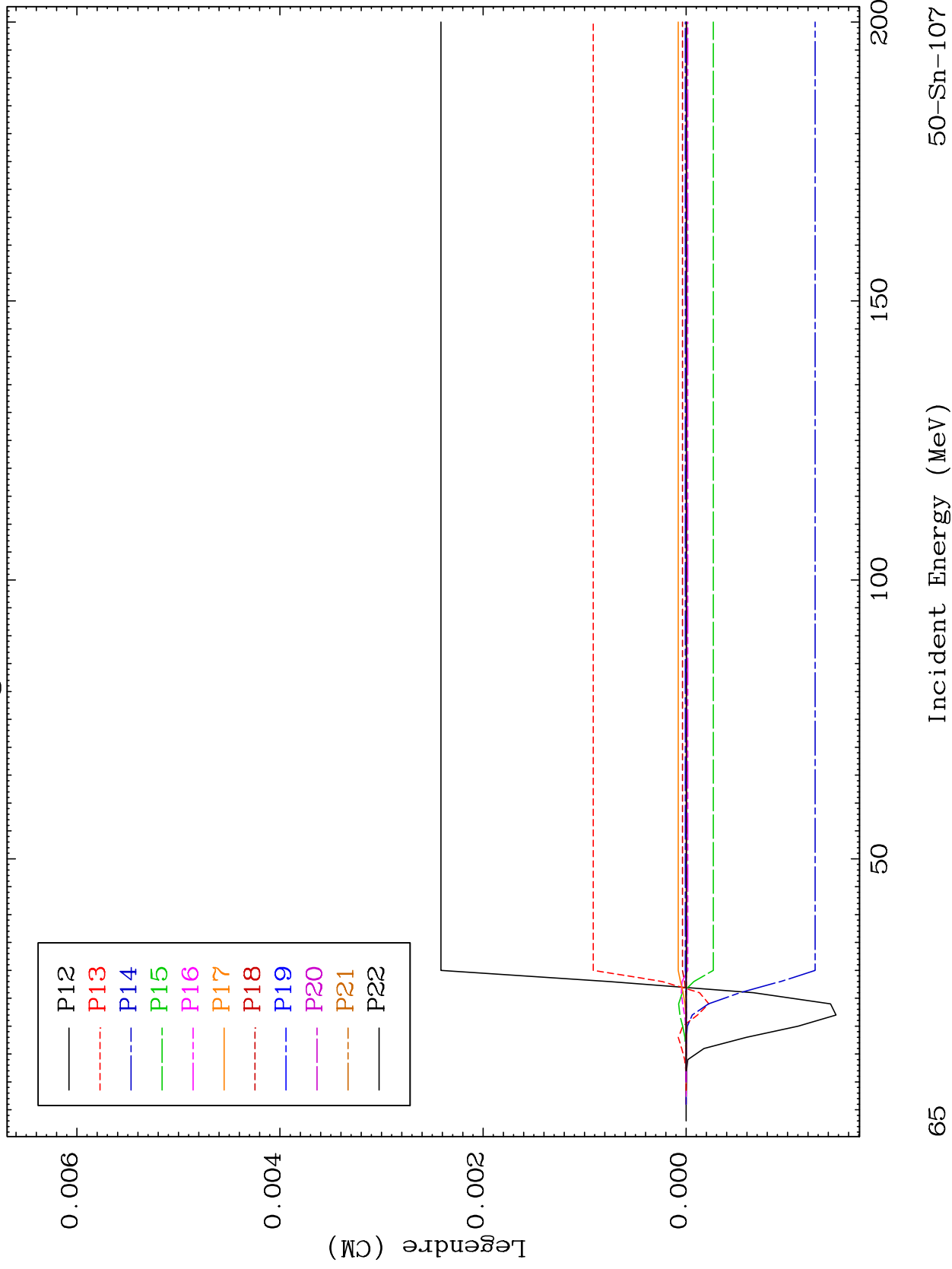




MAT 5010

1.799 MeV (n,n') Level
Legendre Coefficients

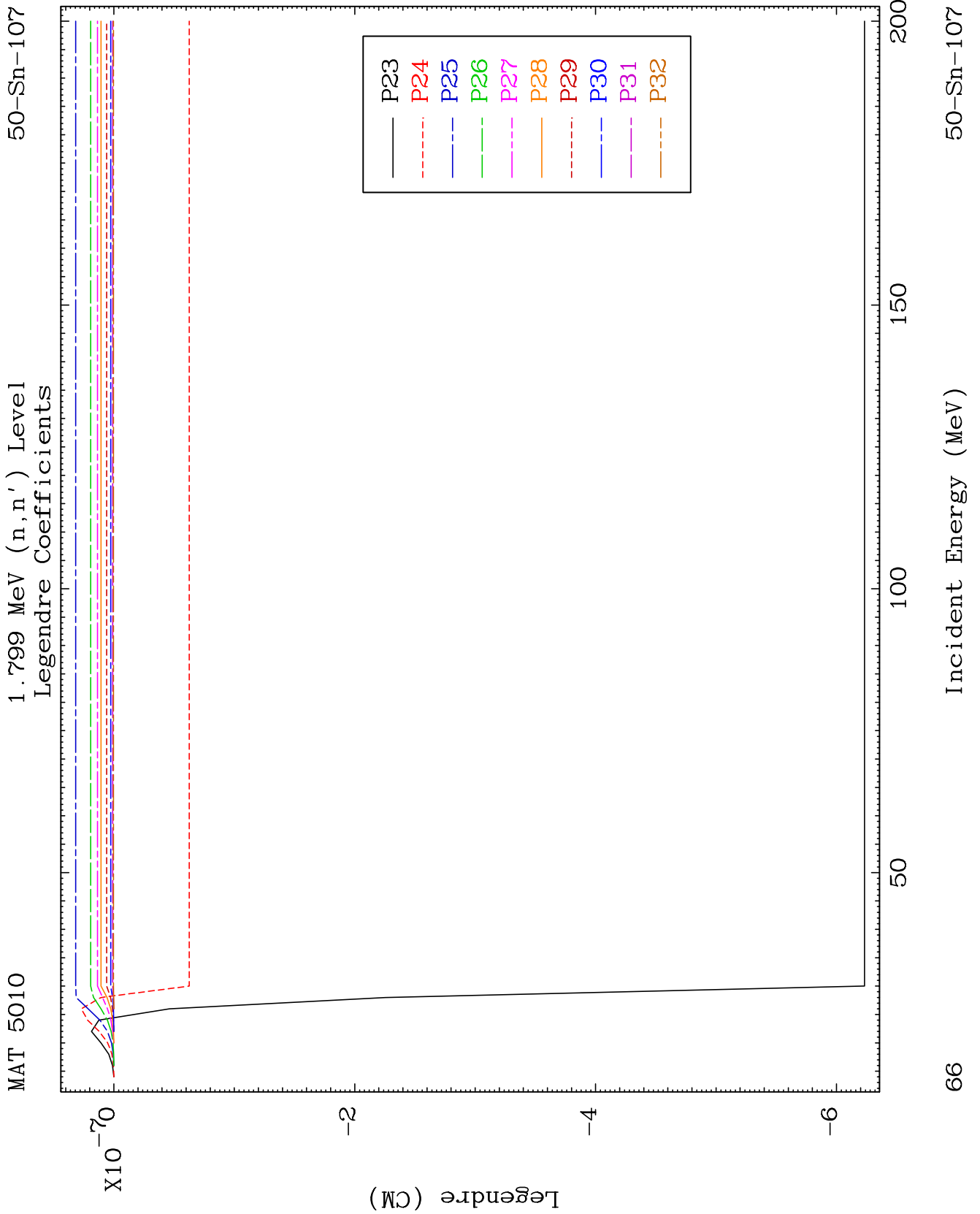
50-Sn-107

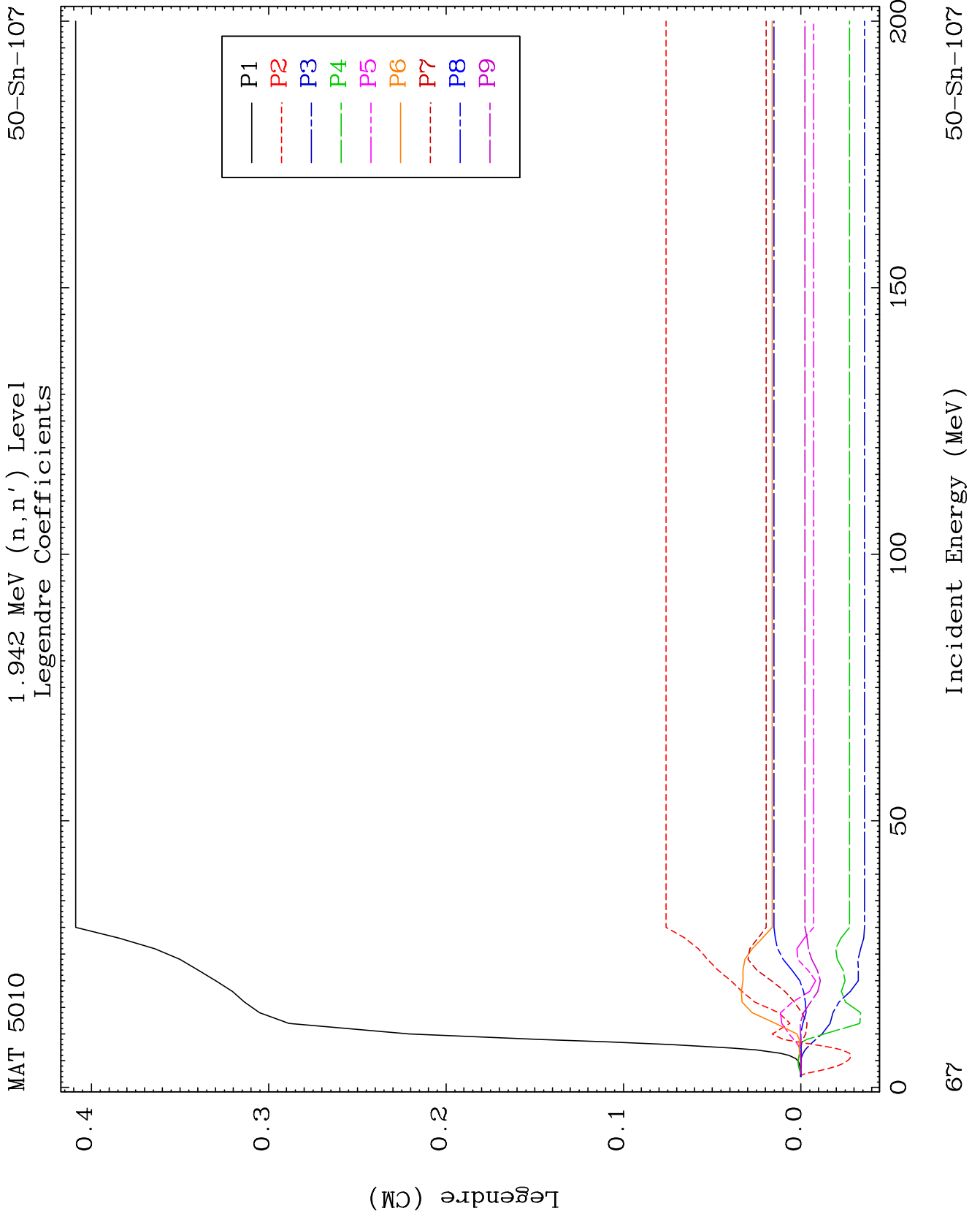


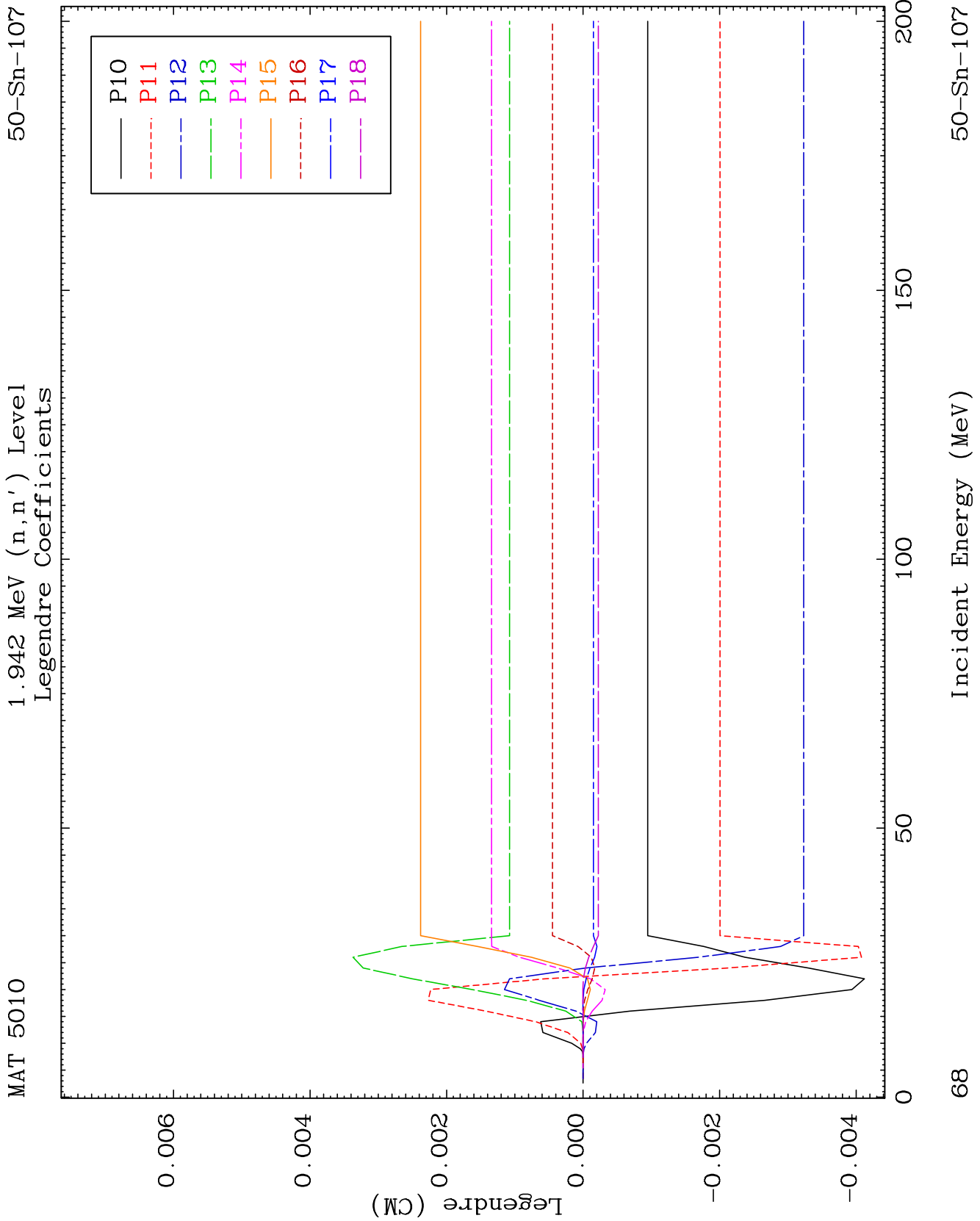
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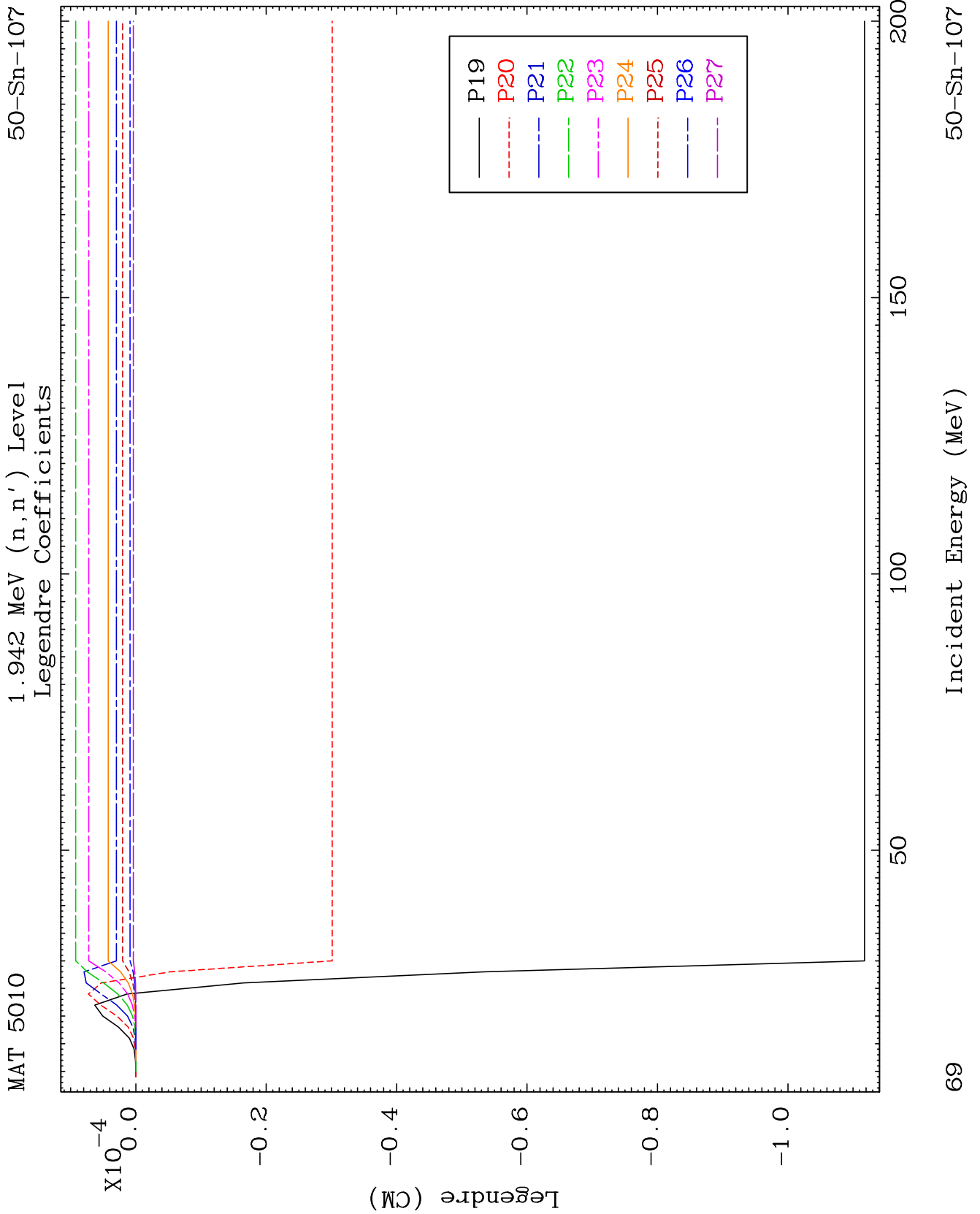
Incident Energy (MeV)

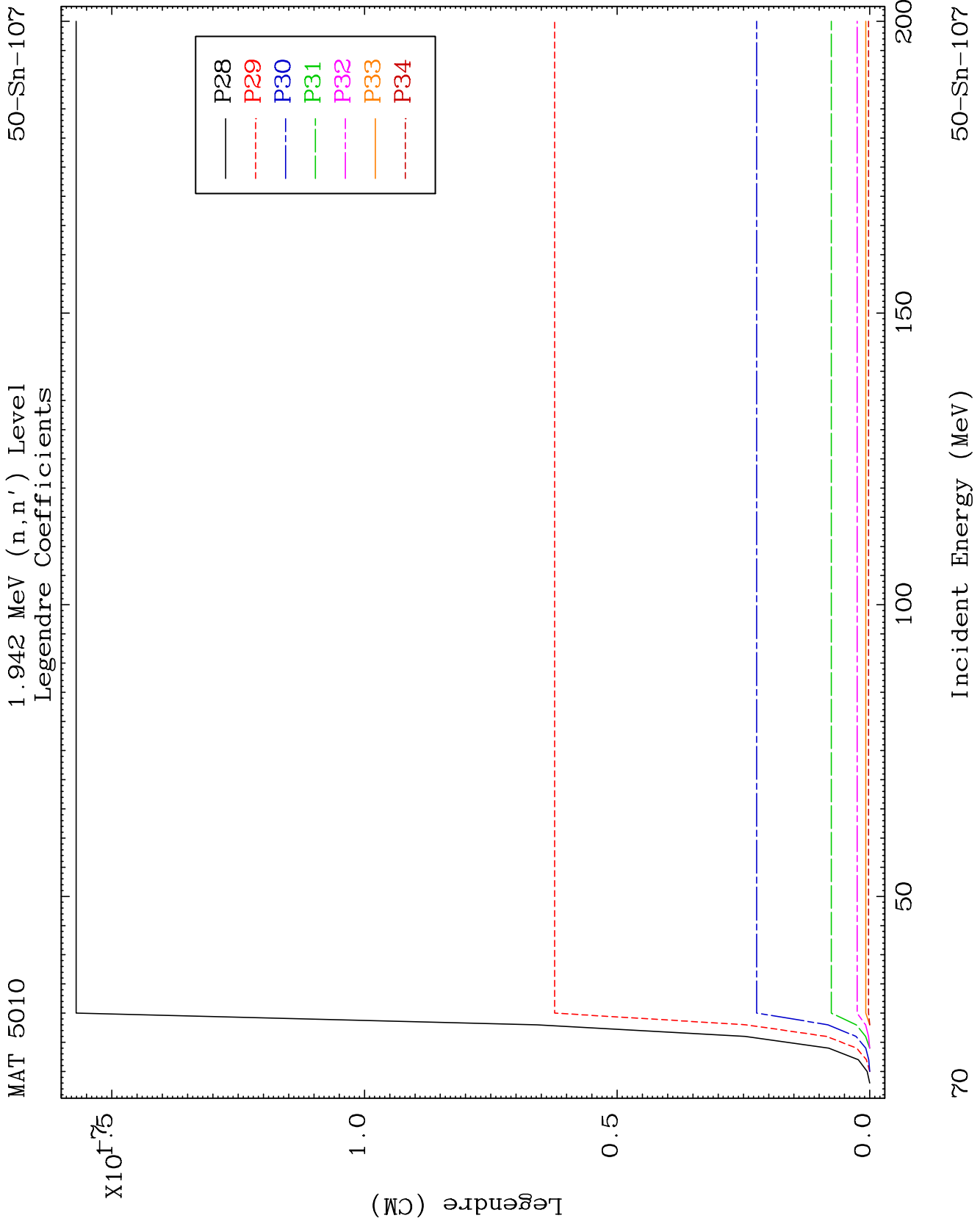
50-Sn-107

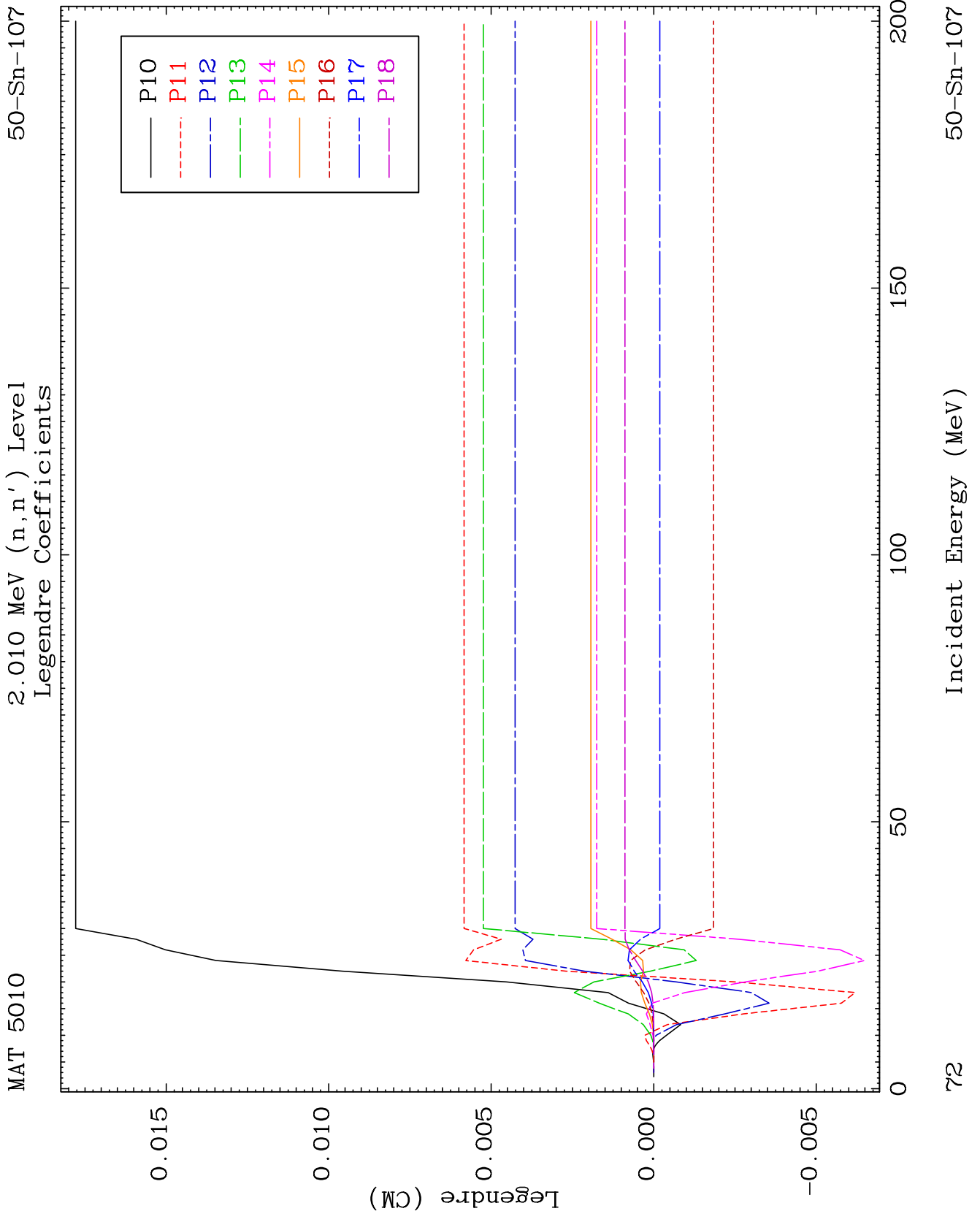


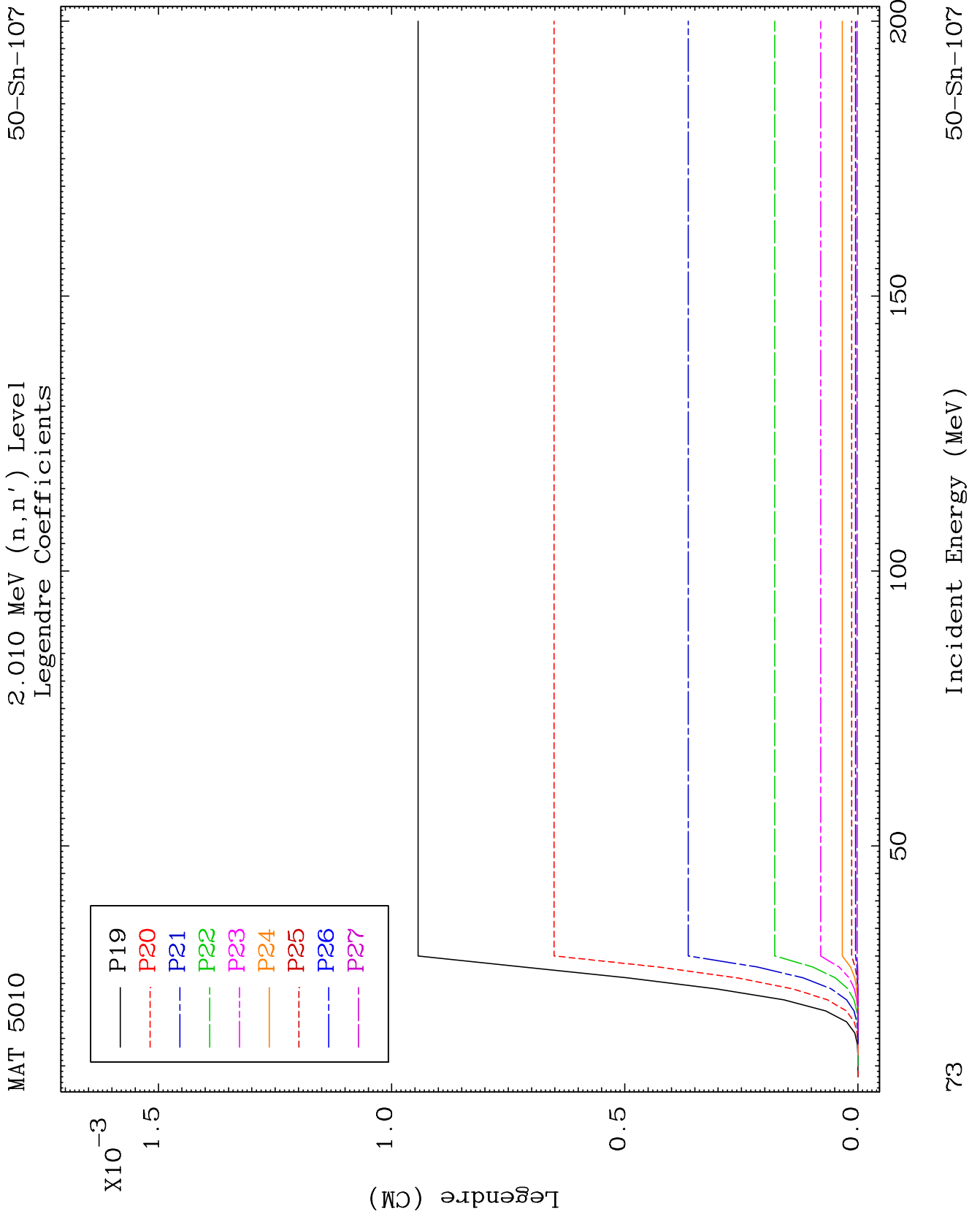


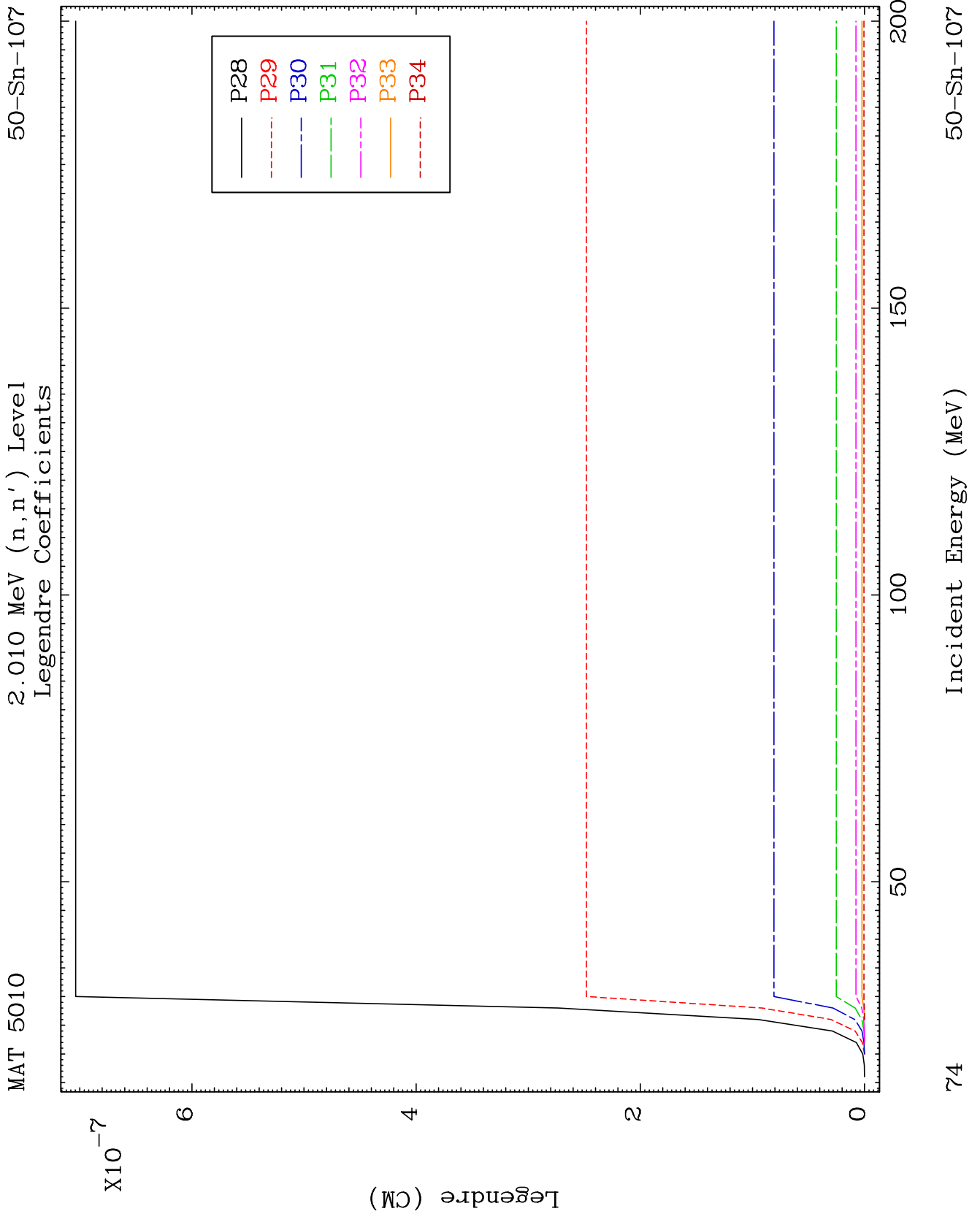


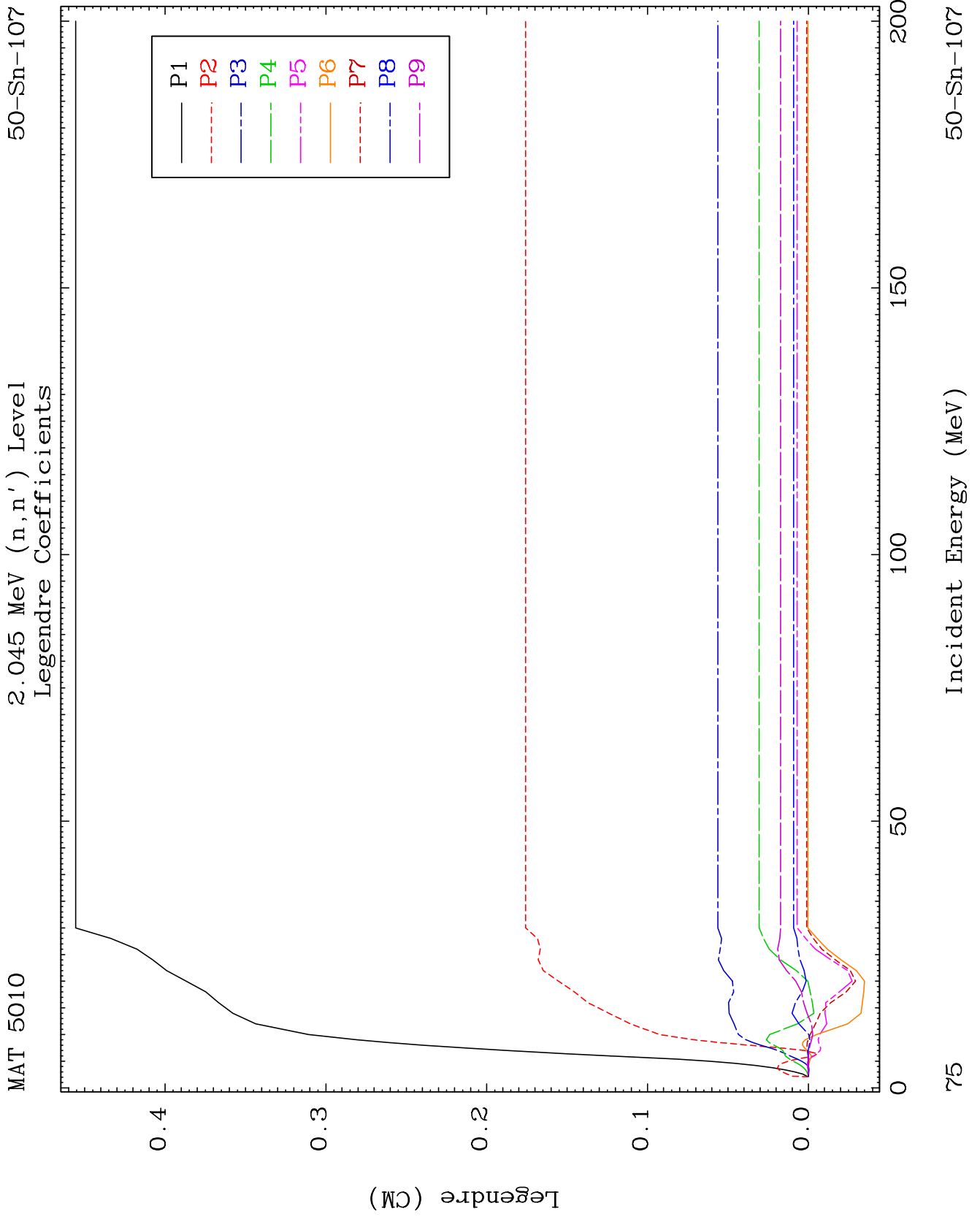


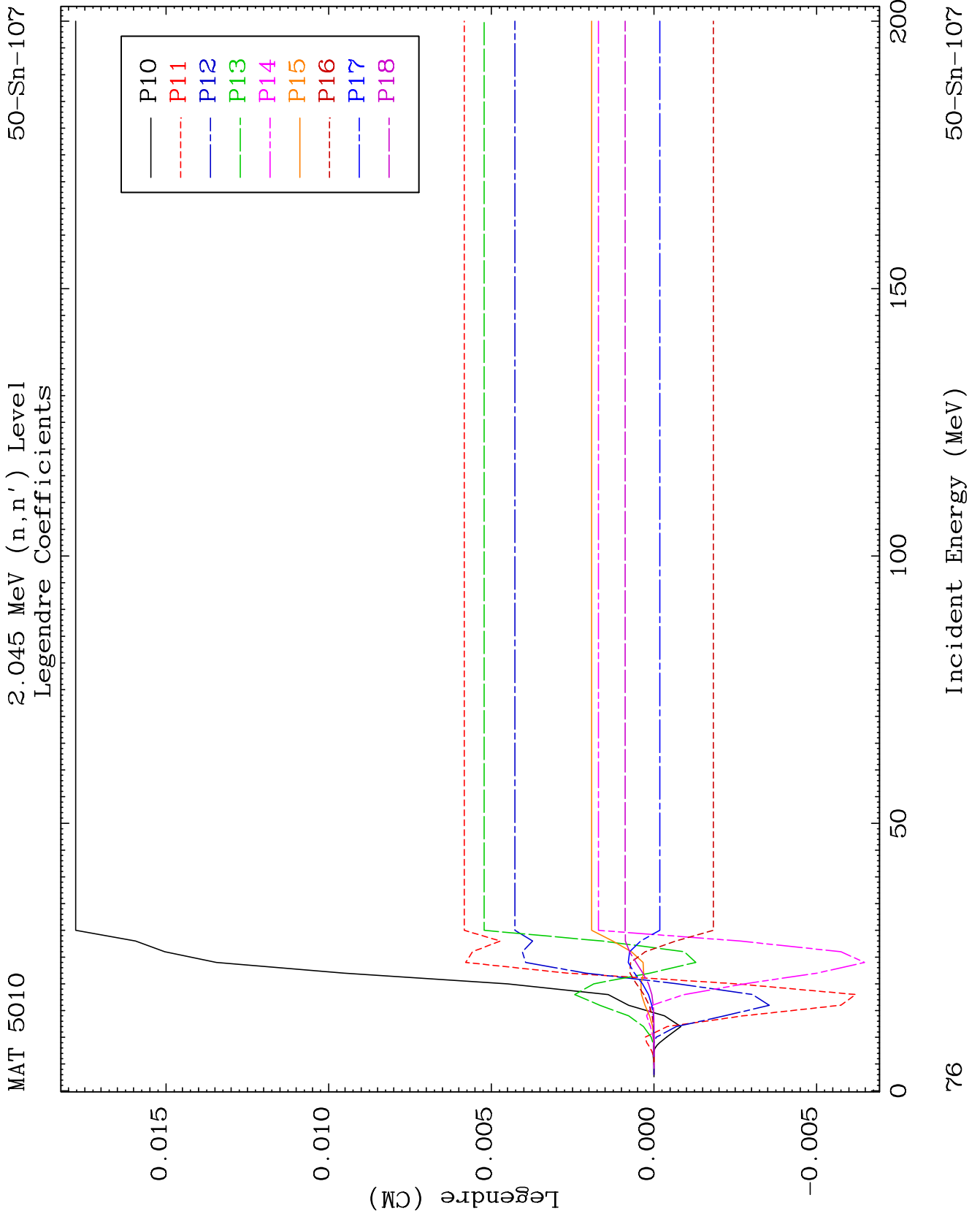


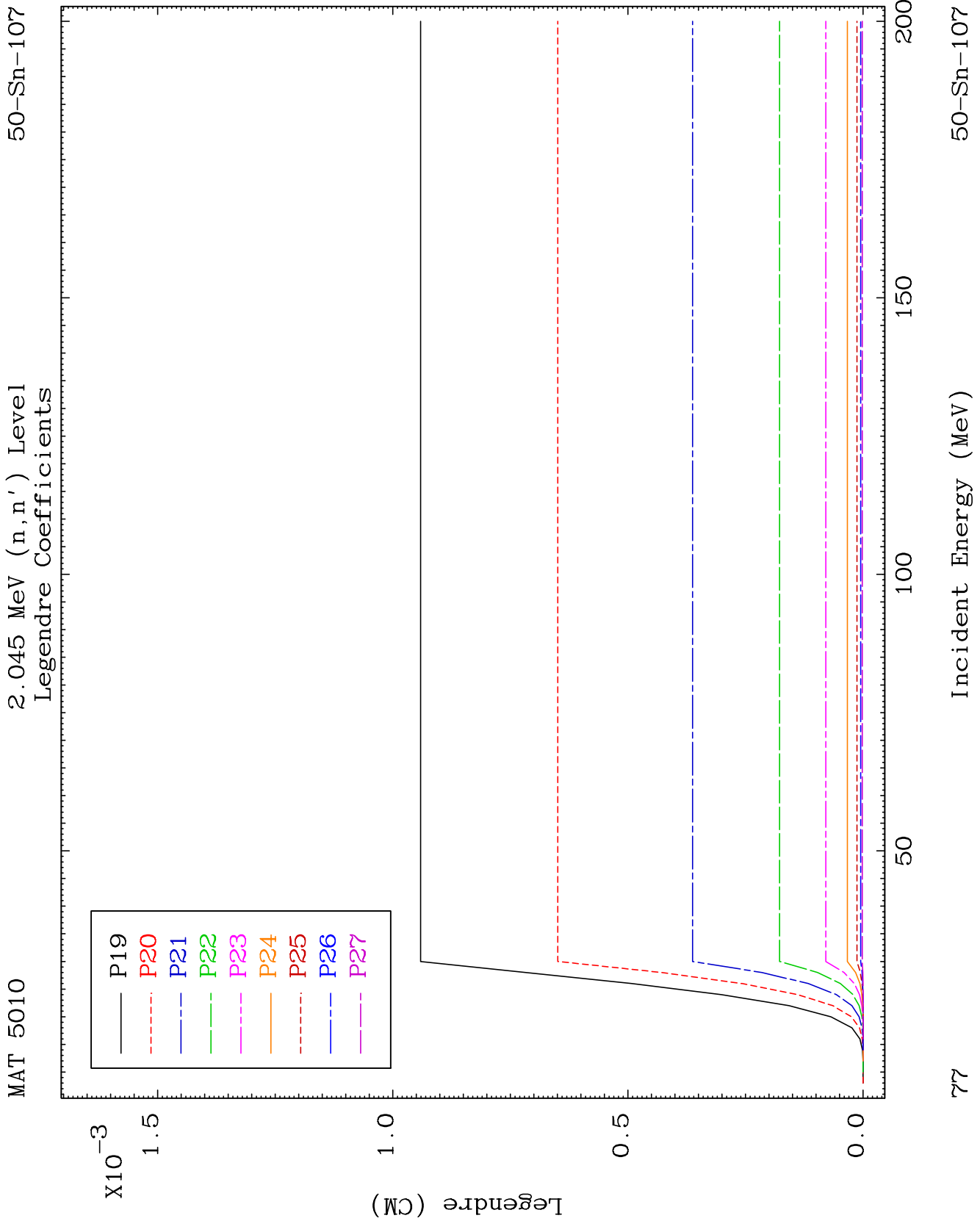


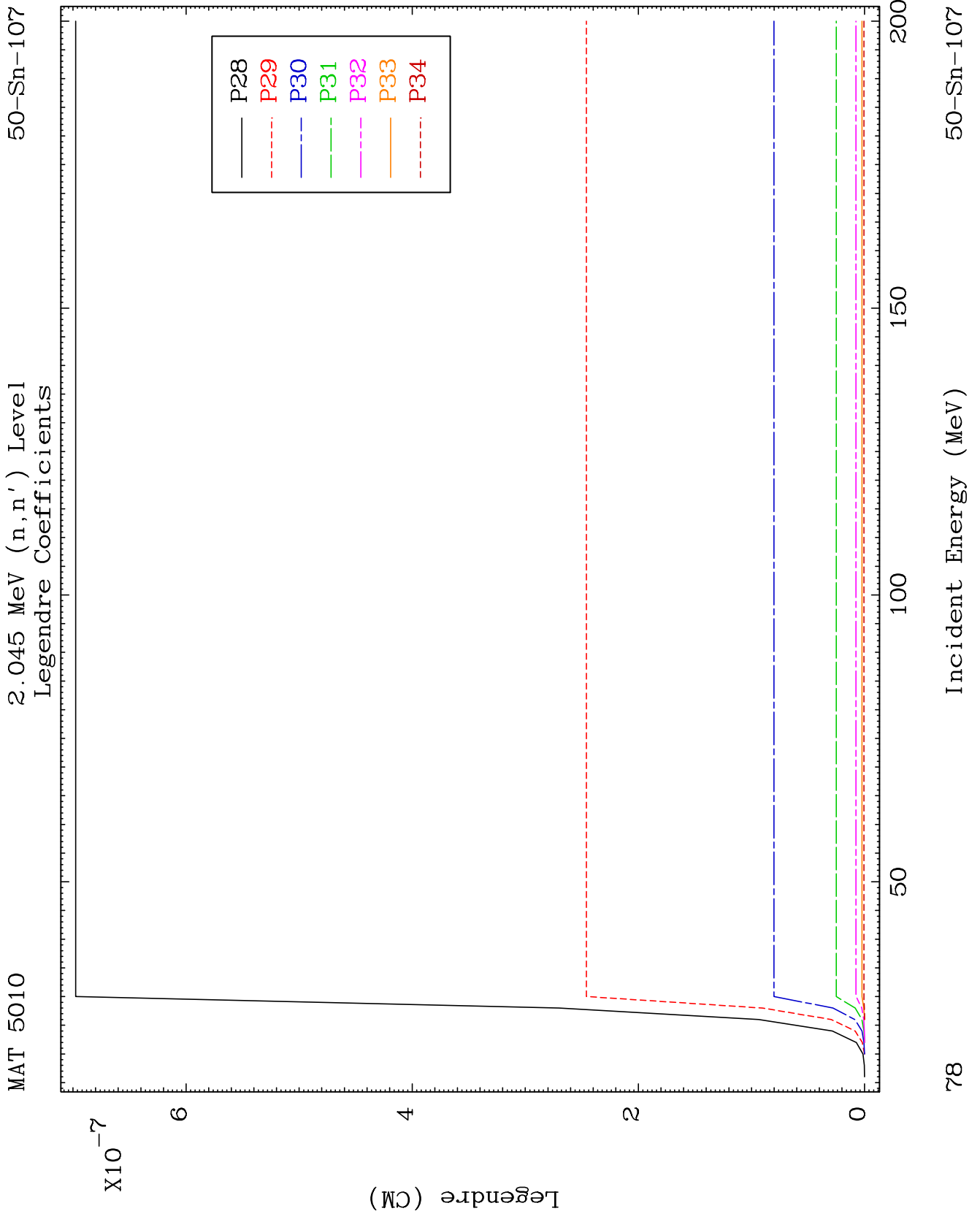


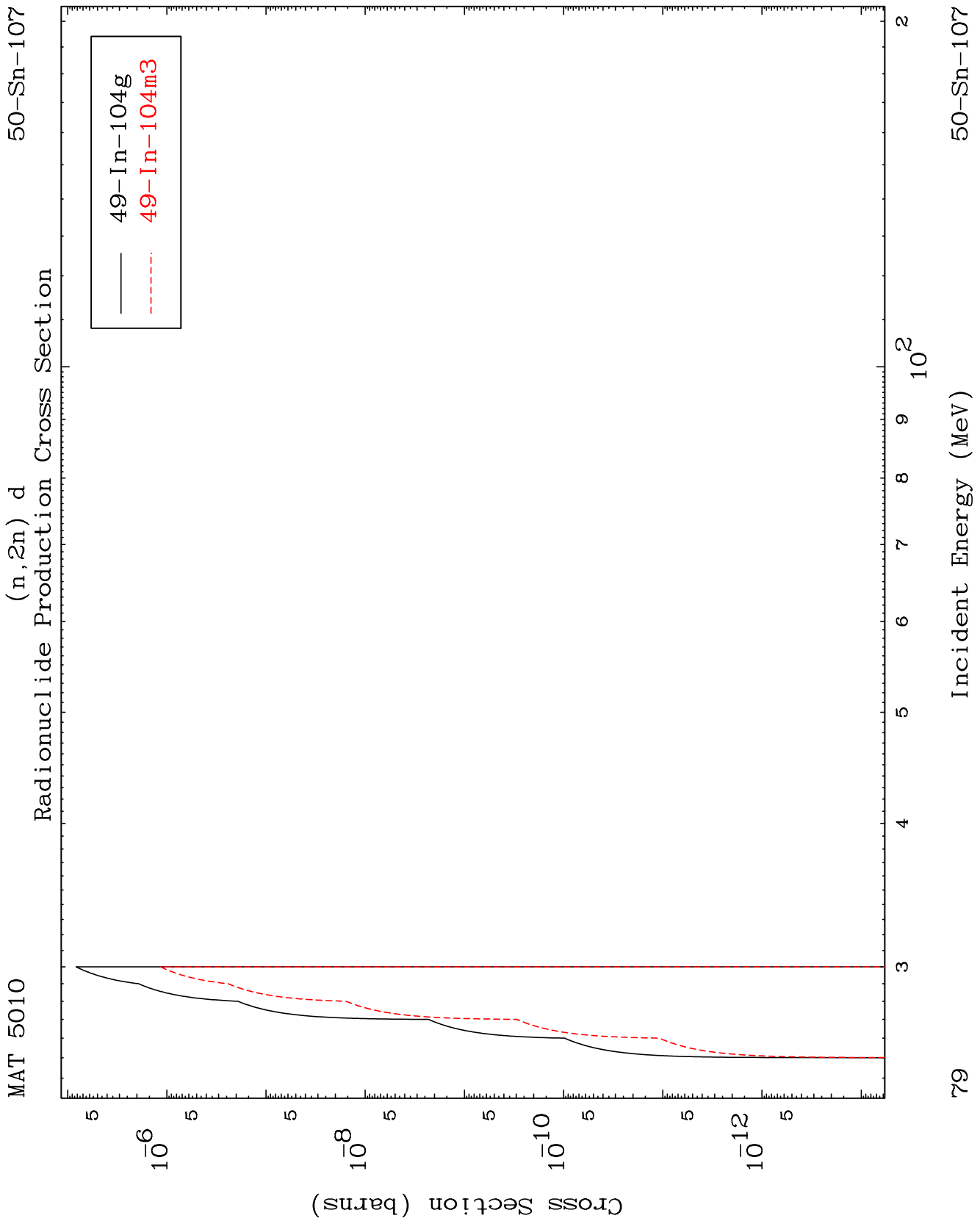










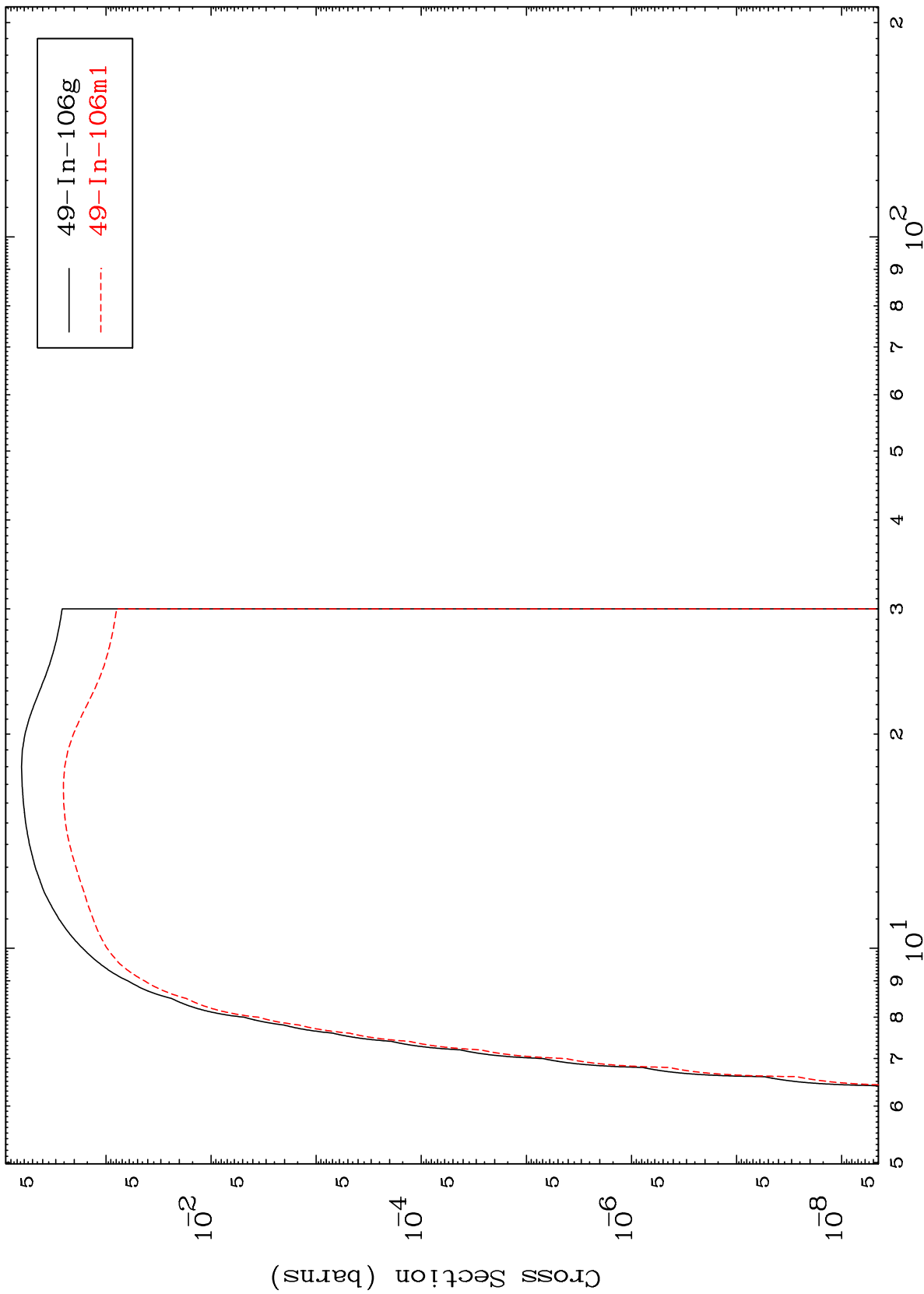


MAT 5010

(n,n') p

50-Sn-107

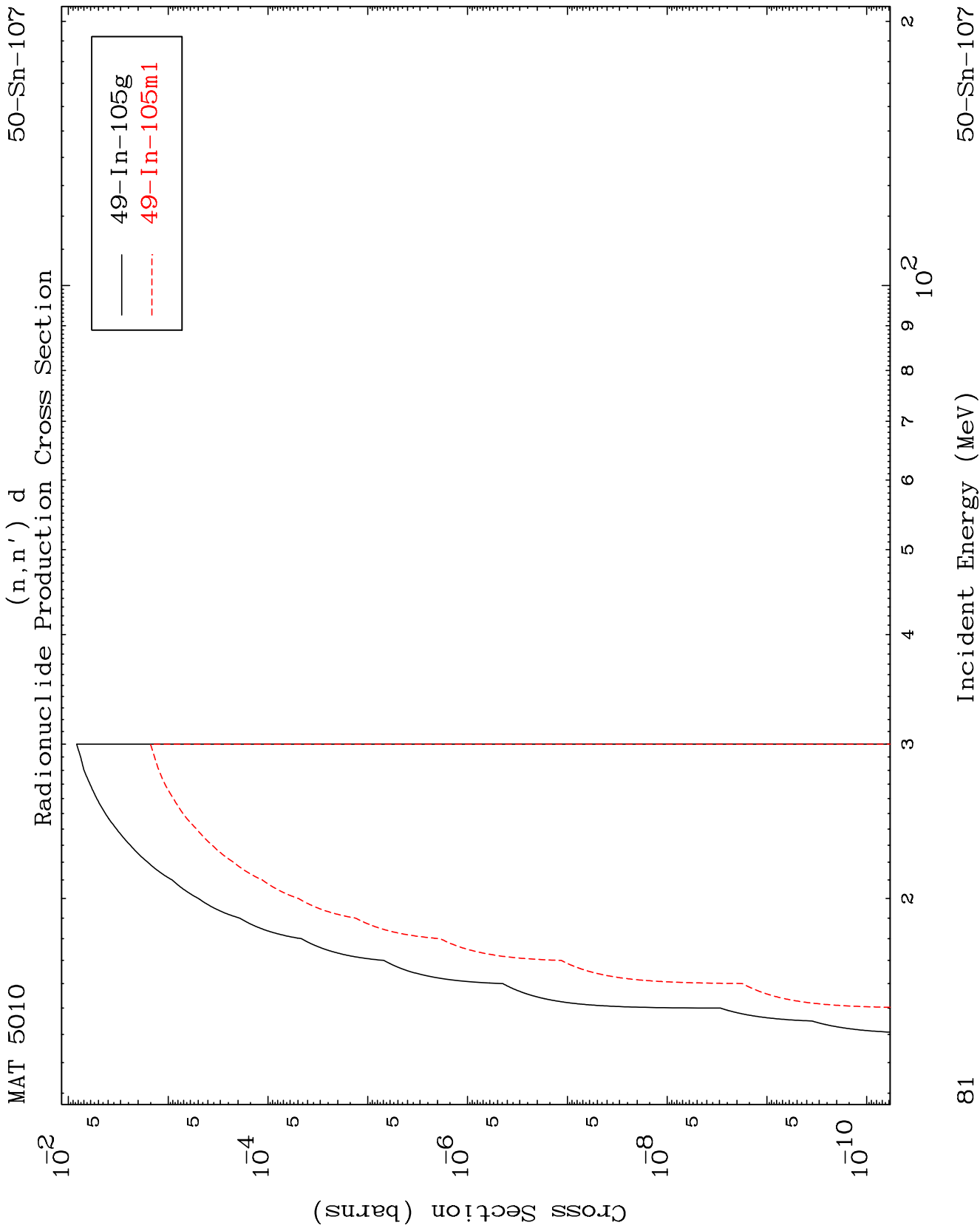
Radionuclide Production Cross Section



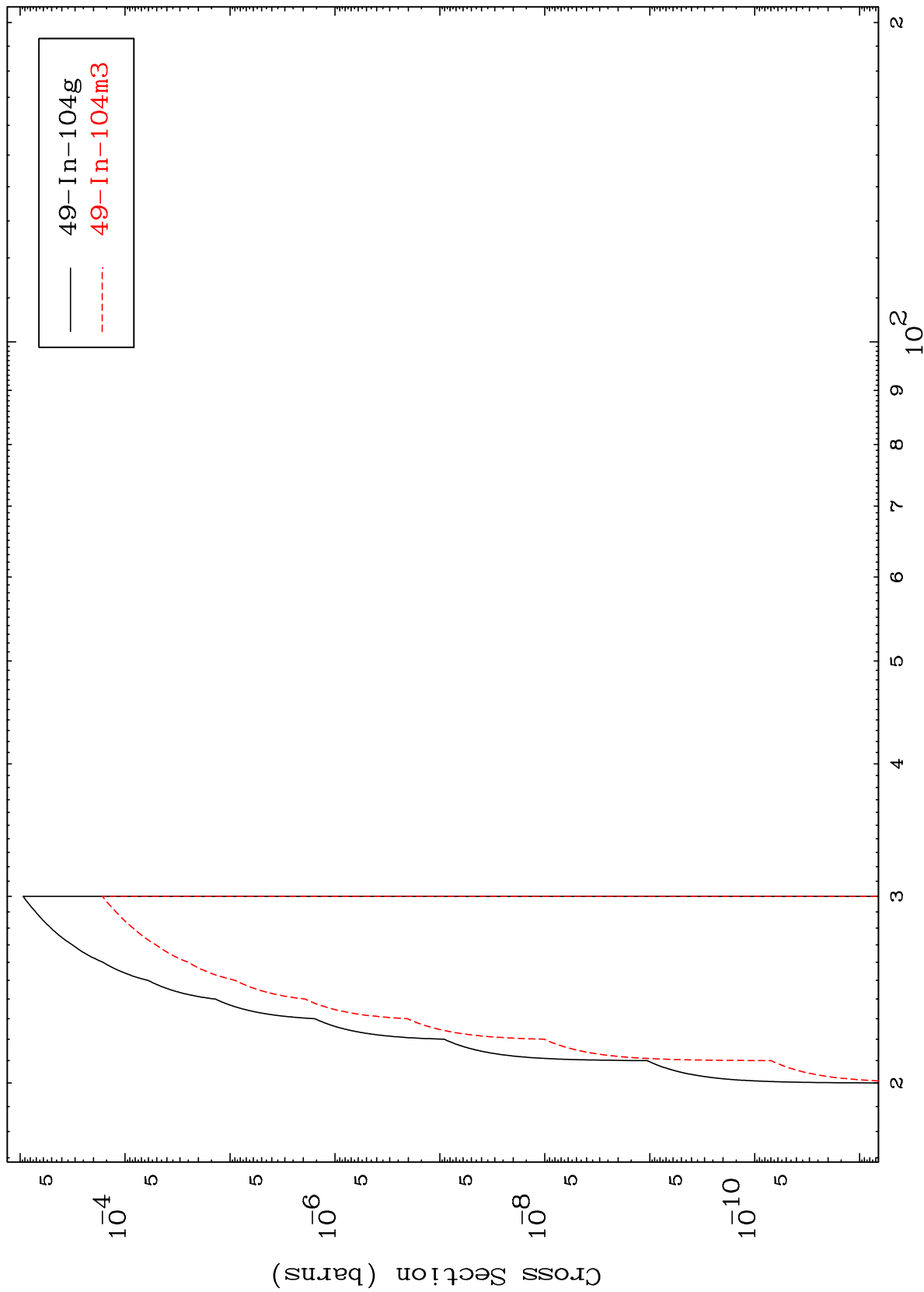
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Incident Energy (MeV)

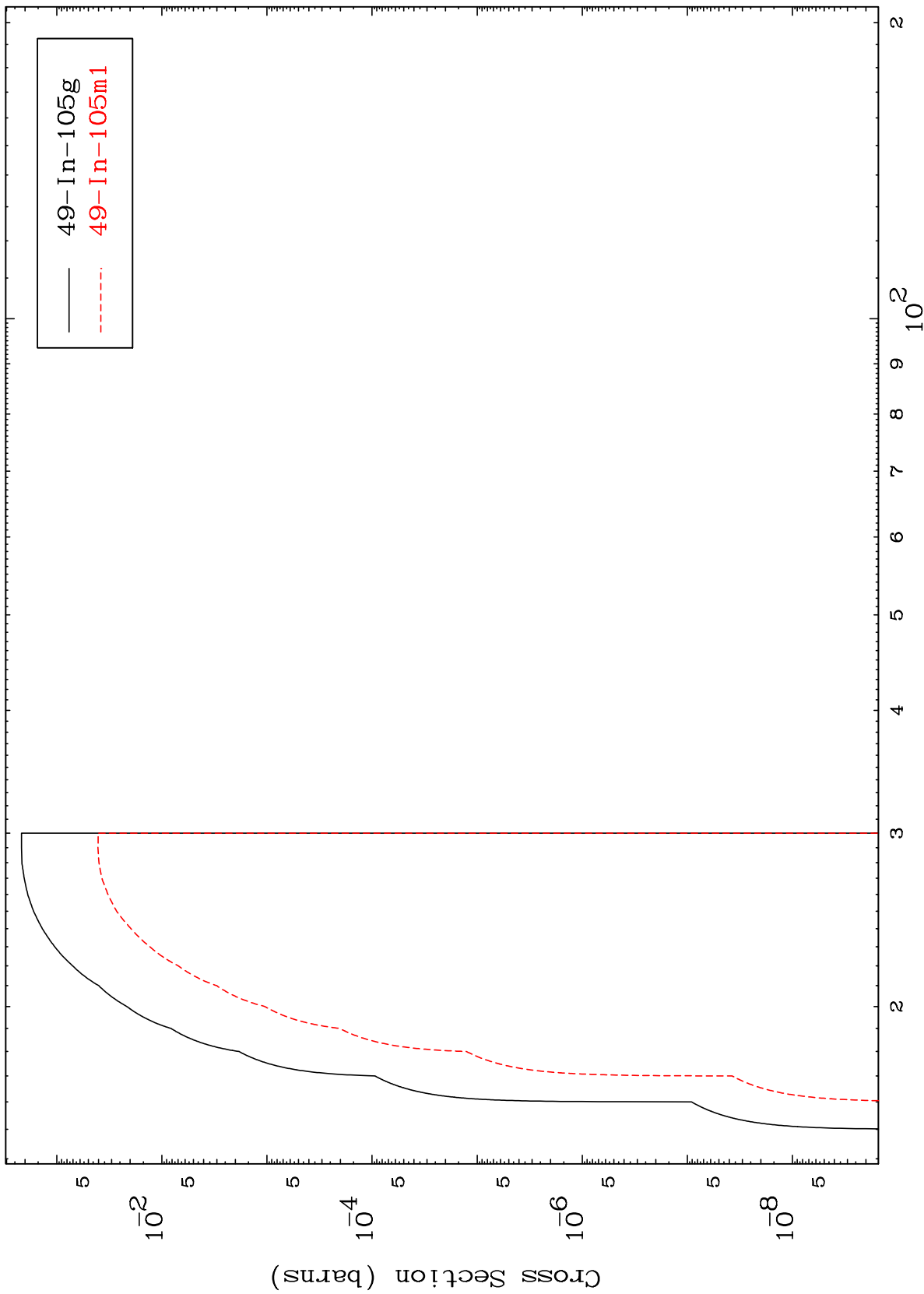
50-Sn-107



Radionuclide Production Cross Section



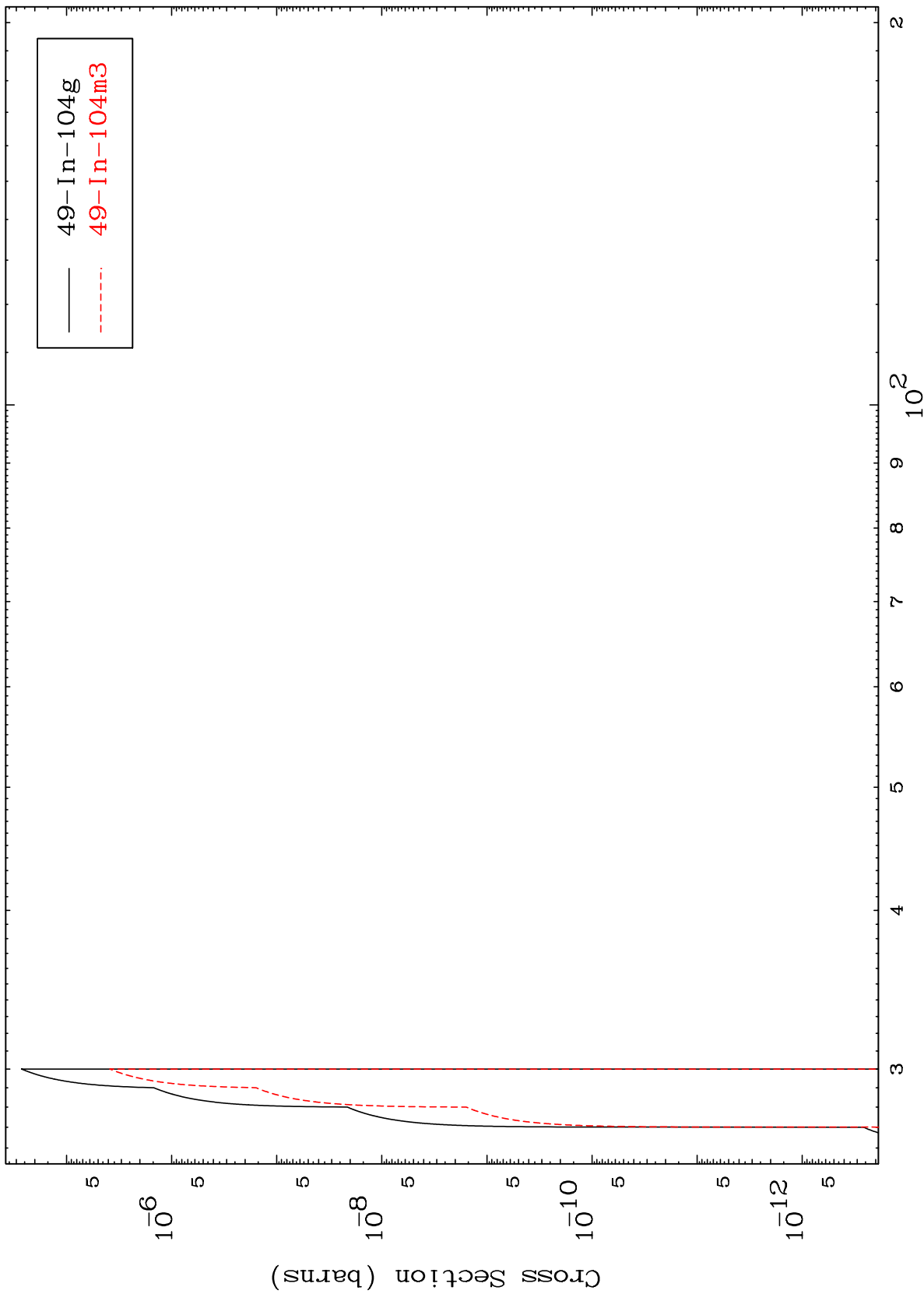
(n,2n) p
Radionuclide Production Cross Section



MAT 5010

50-Sn-107

(n,3n) p
Radionuclide Production Cross Section



50-Sn-107

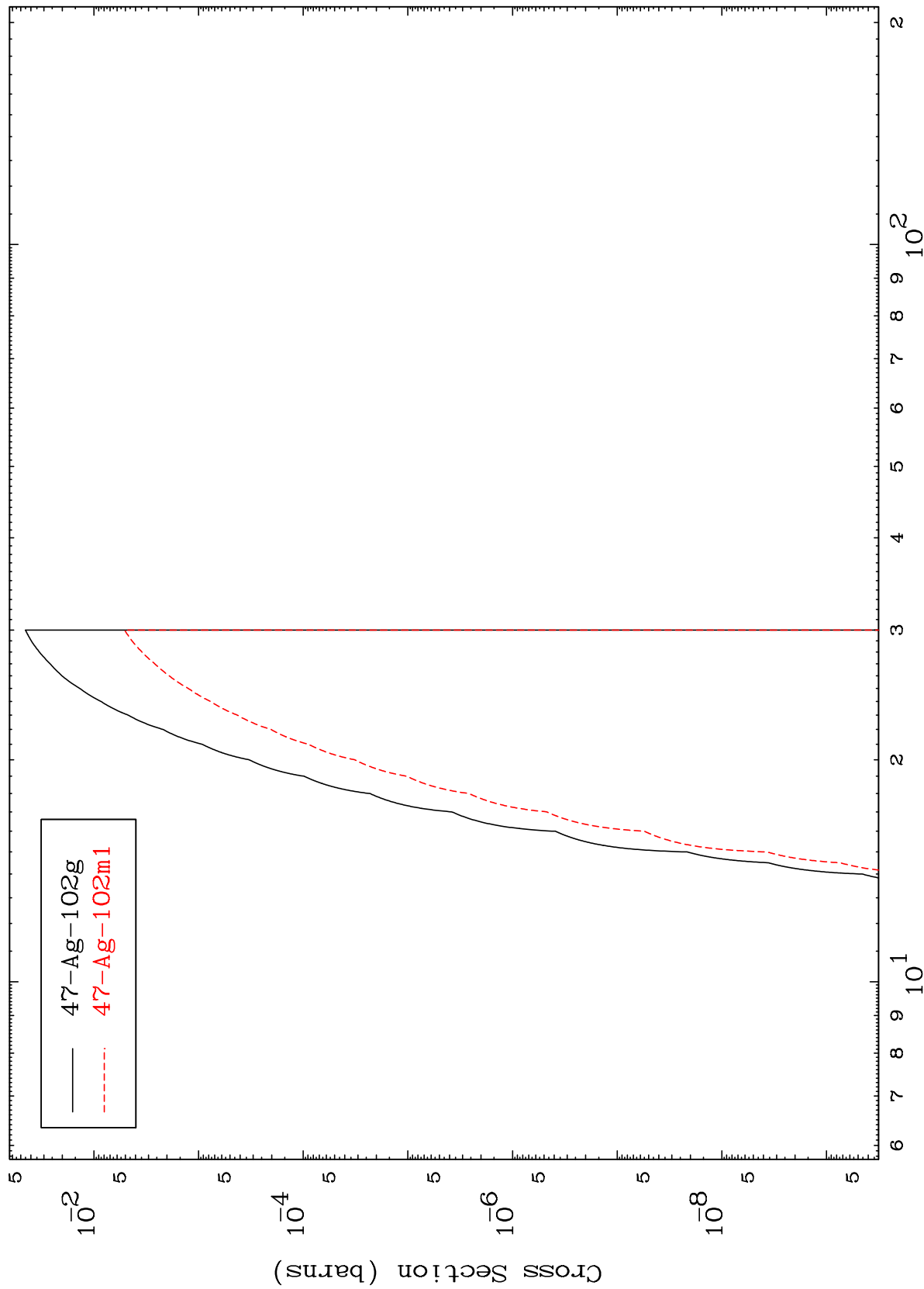
Incident Energy (MeV)

84

MAT 5010

50-Sn-107

(n,n') p α
Radionuclide Production Cross Section



— 47-Ag-102g
- - - 47-Ag-102m1

85

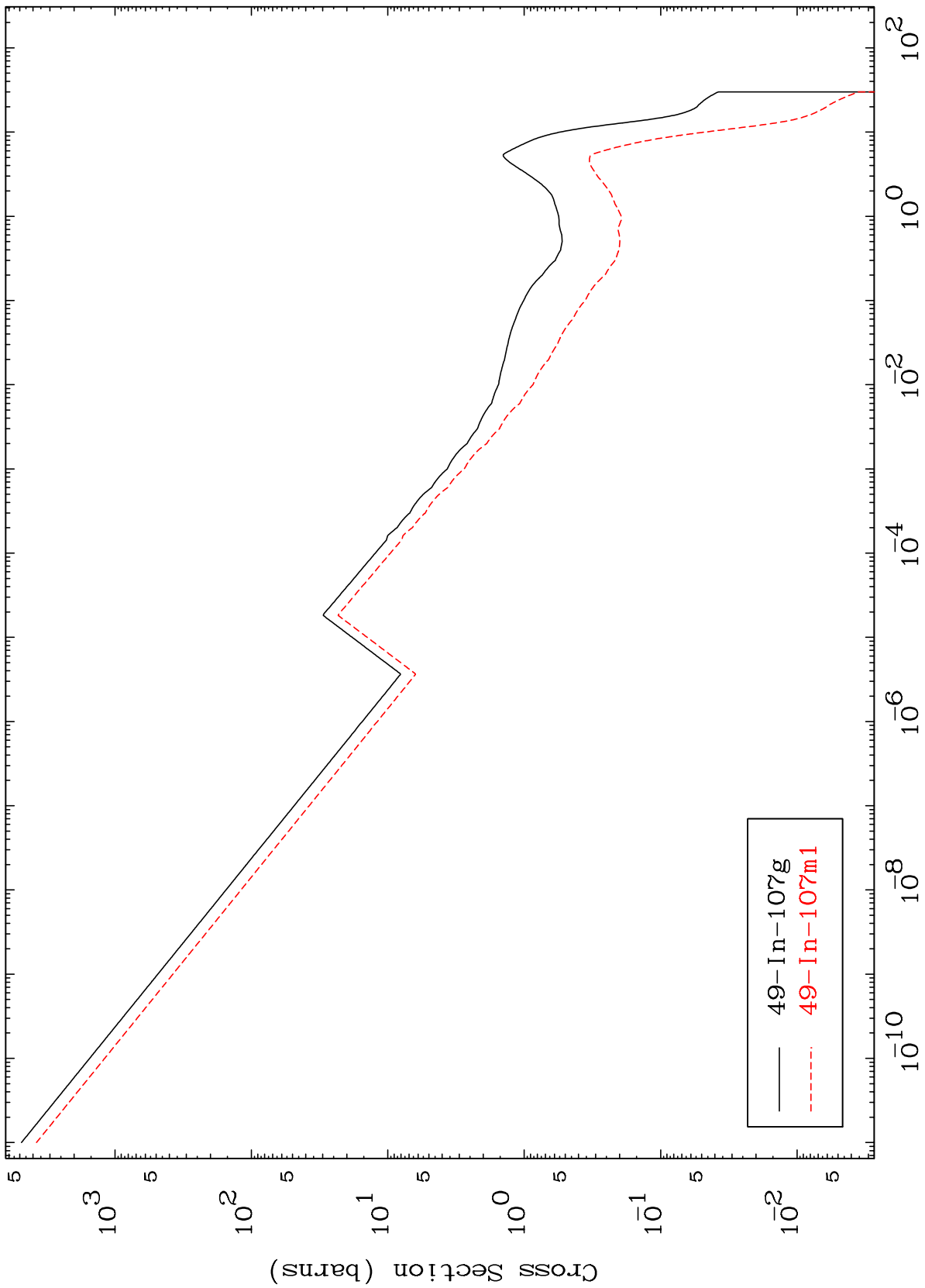
Incident Energy (MeV)

50-Sn-107

MAT 5010

50-Sn-107

(n,p)
Radionuclide Production Cross Section

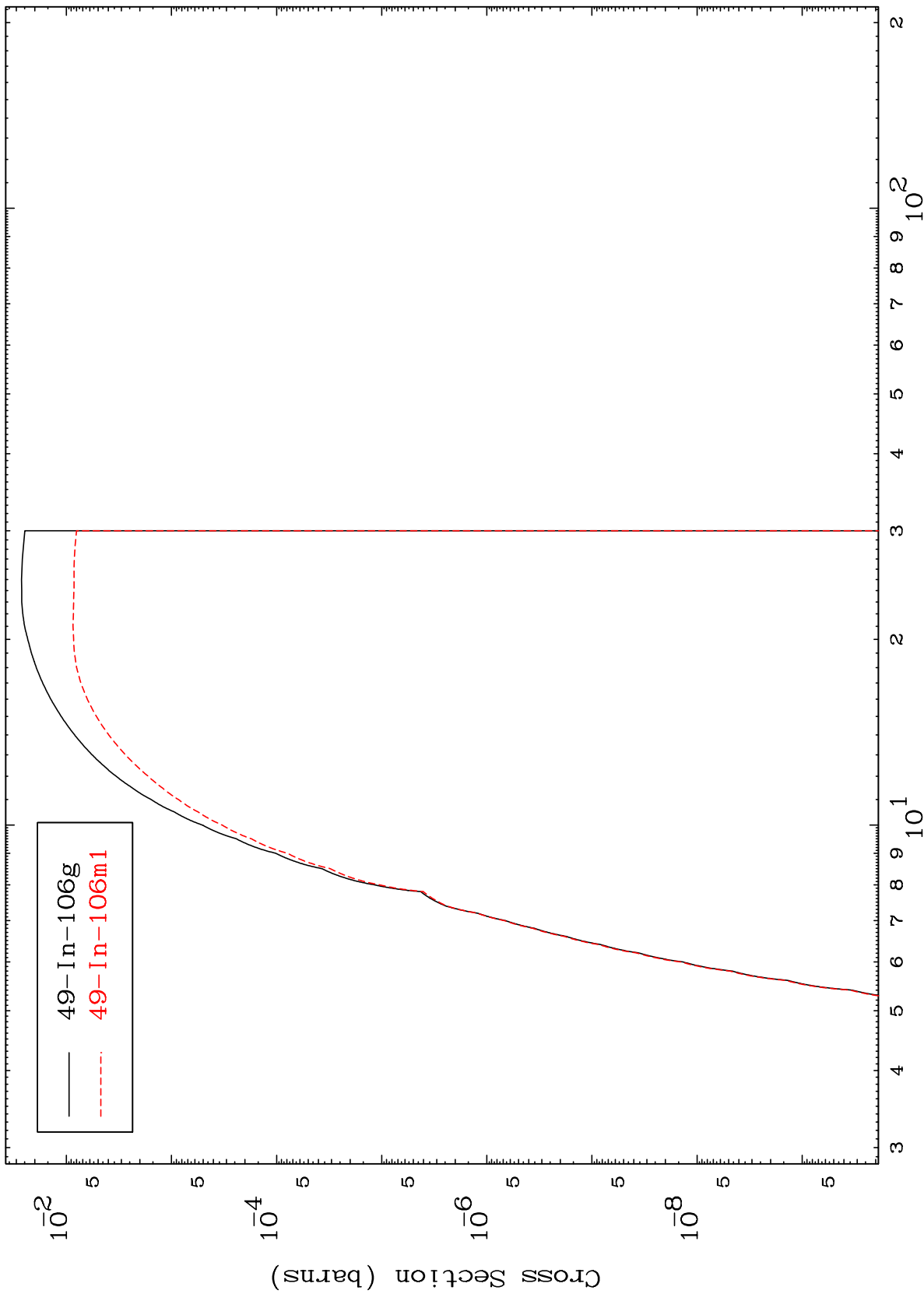


— ^{107g}In
- - - $^{107m1}\text{In}$

MAT 5010

50-Sn-107

(n,d)
Radionuclide Production Cross Section



— 49-In-106g
- - - 49-In-106m1

50-Sn-107

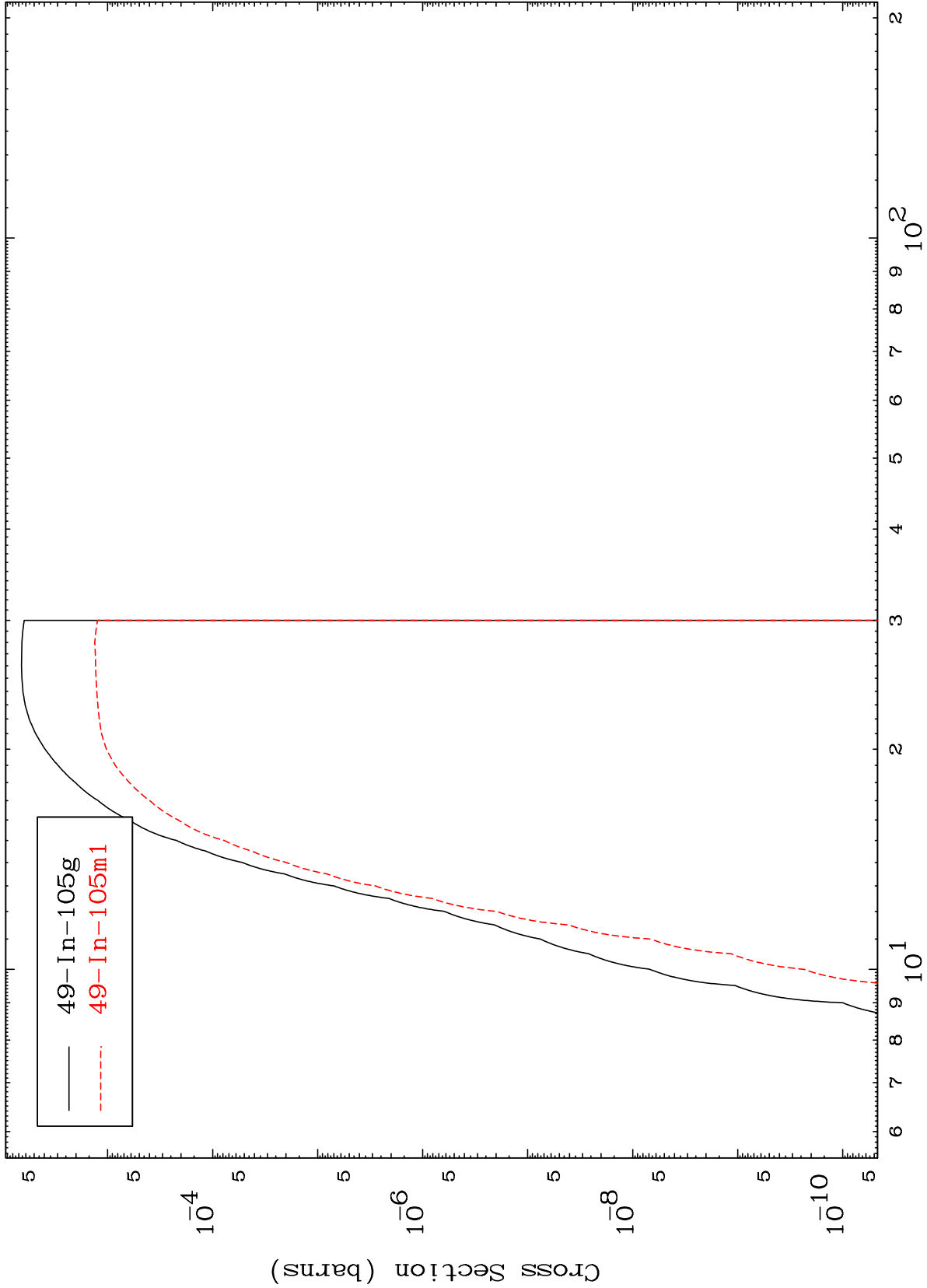
Incident Energy (MeV)

87

MAT 5010

50-Sn-107

(n,t)
Radionuclide Production Cross Section



— 49-In-105g
- - - 49-In-105m1

50-Sn-107

Incident Energy (MeV)

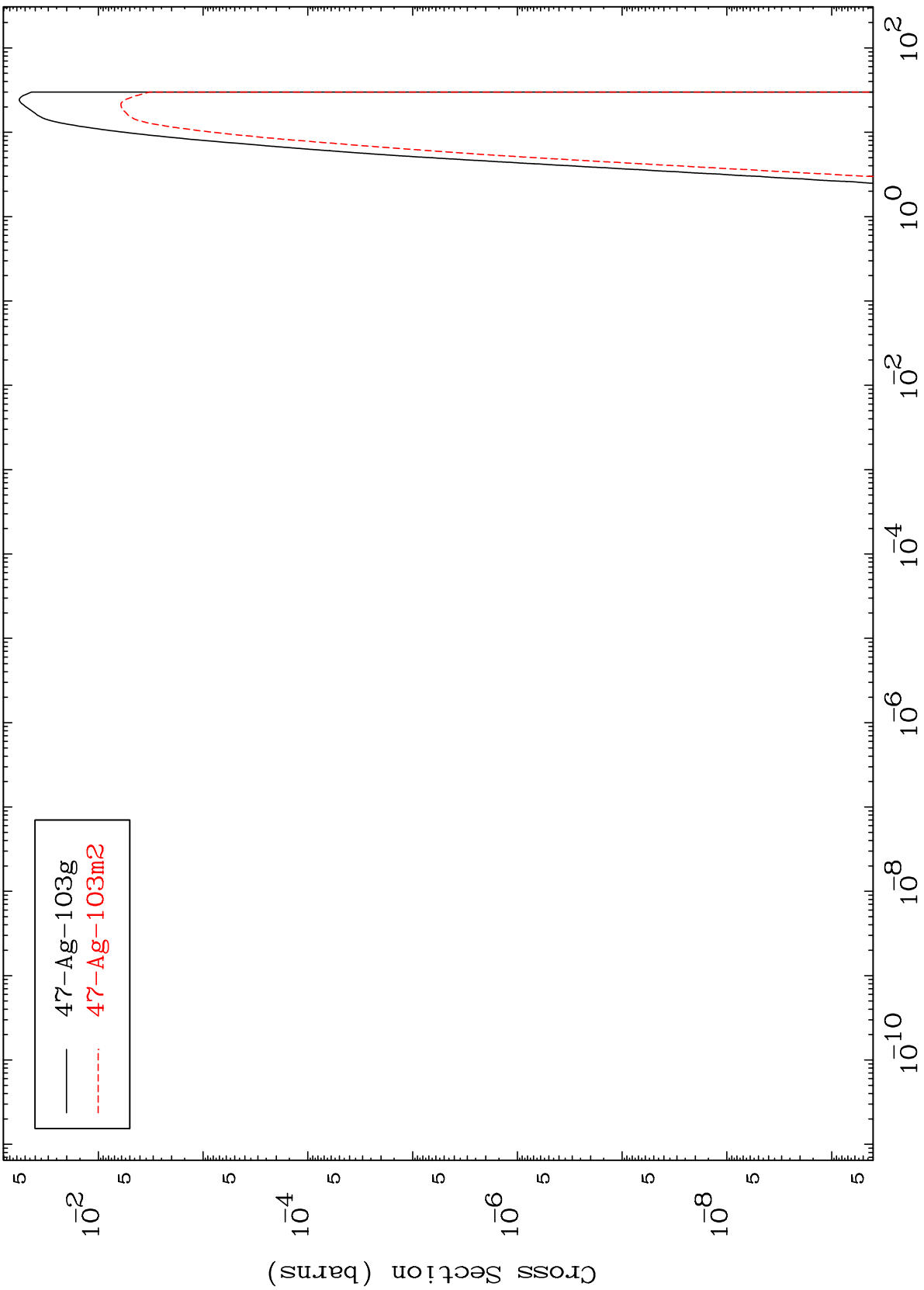
88

MAT 5010

(n,p) α

50-Sn-107

Radionuclide Production Cross Section



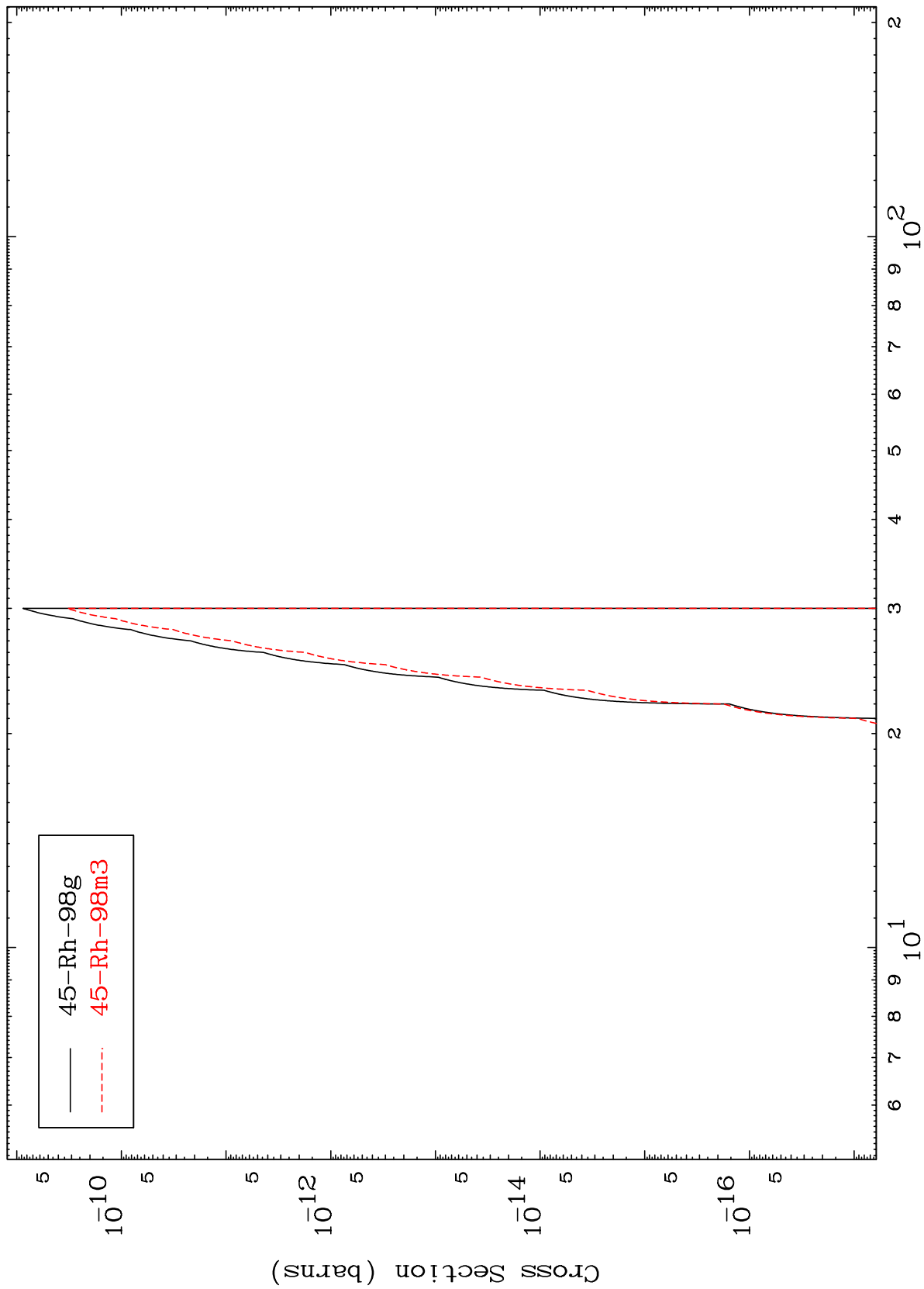
— 47-Ag-103g
- - - 47-Ag-103m2

MAT 5010

(n,d) 2α

50-Sn-107

Radionuclide Production Cross Section



45-Rh-98g
45-Rh-98m3

90

Incident Energy (MeV)

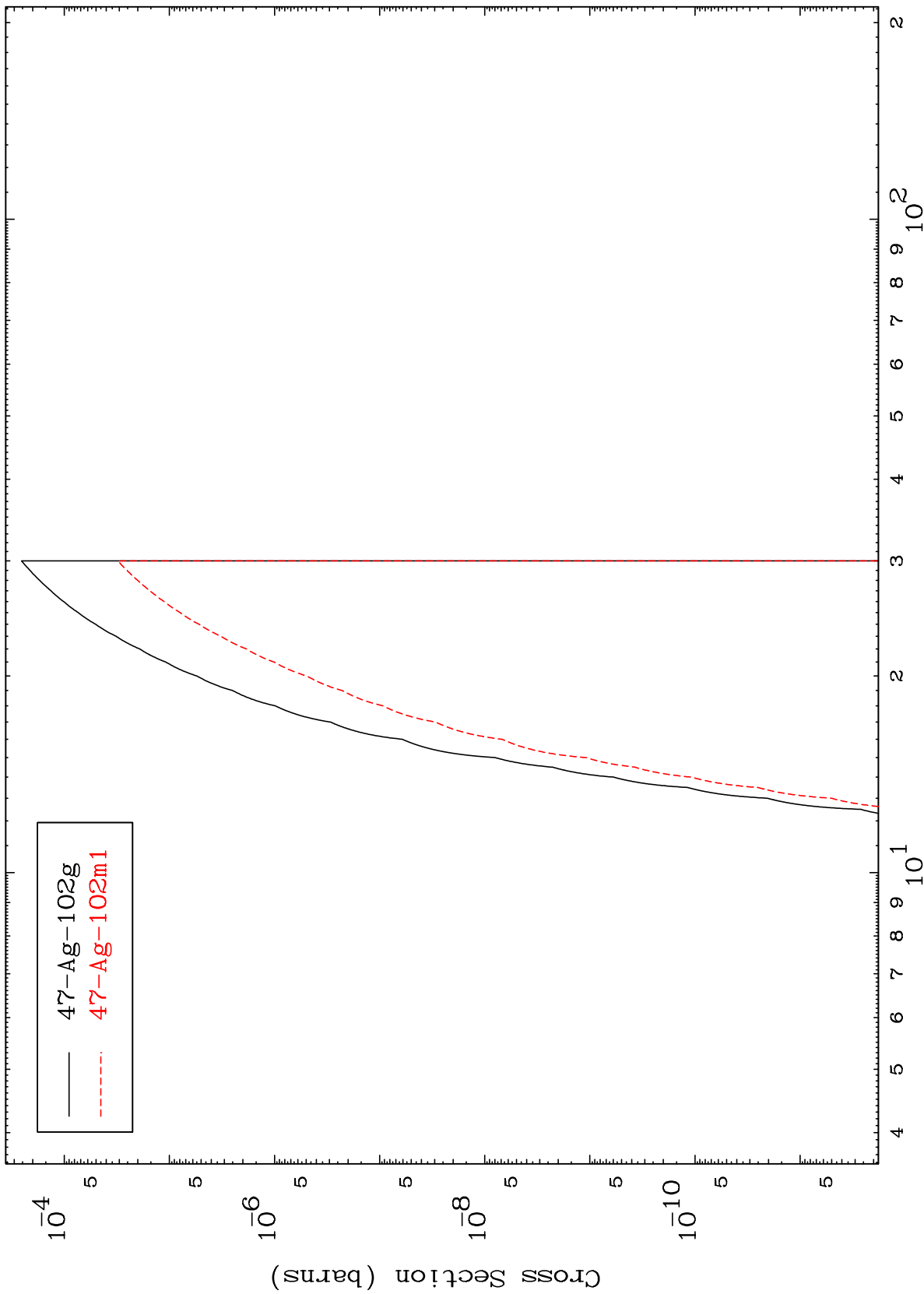
50-Sn-107

MAT 5010

(n,d) α

50-Sn-107

Radionuclide Production Cross Section



— 47-Ag-102g
- - - 47-Ag-102m1