

Program EVALPLOT
(Version 2015-2)

by

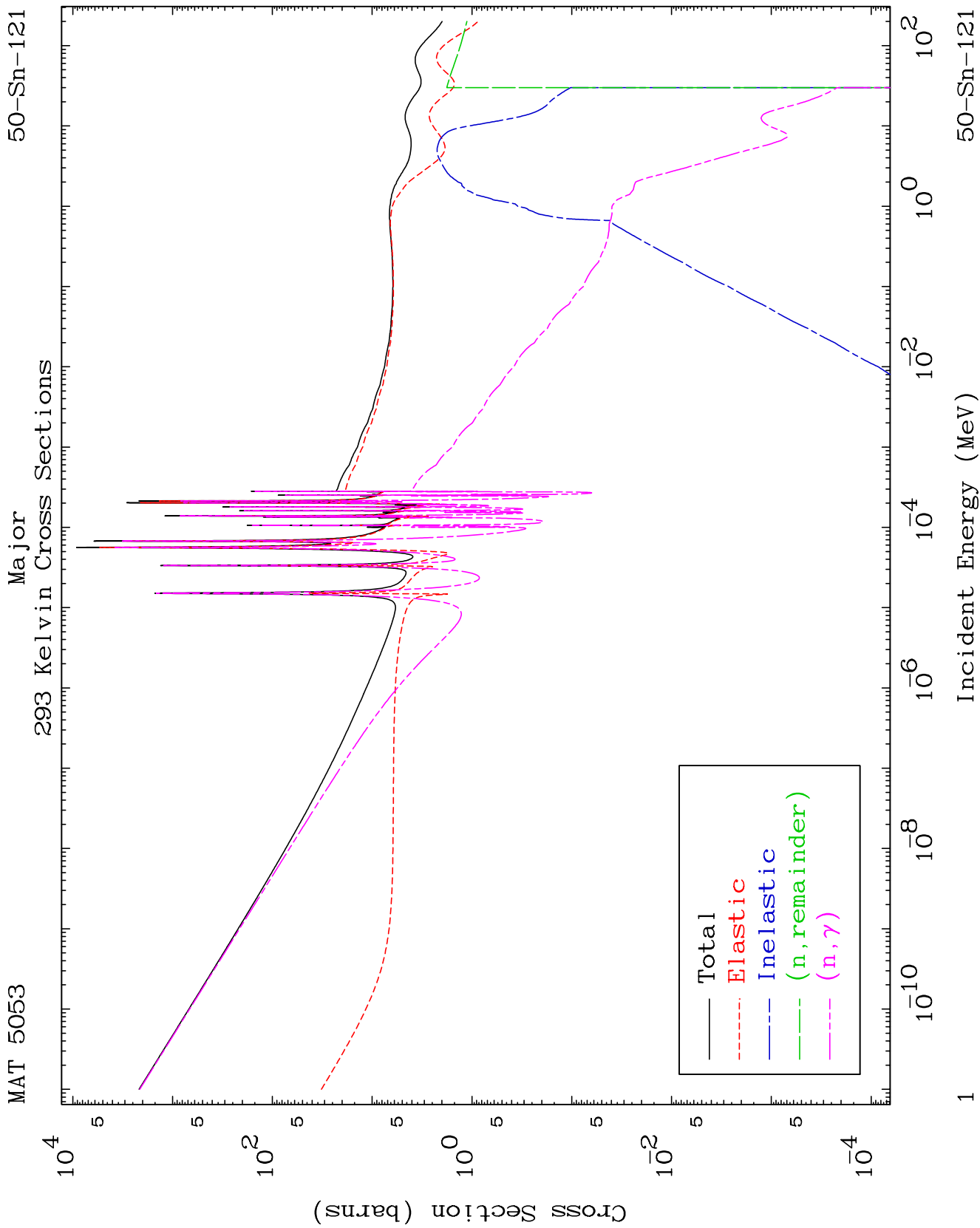
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(Present Contact Information)

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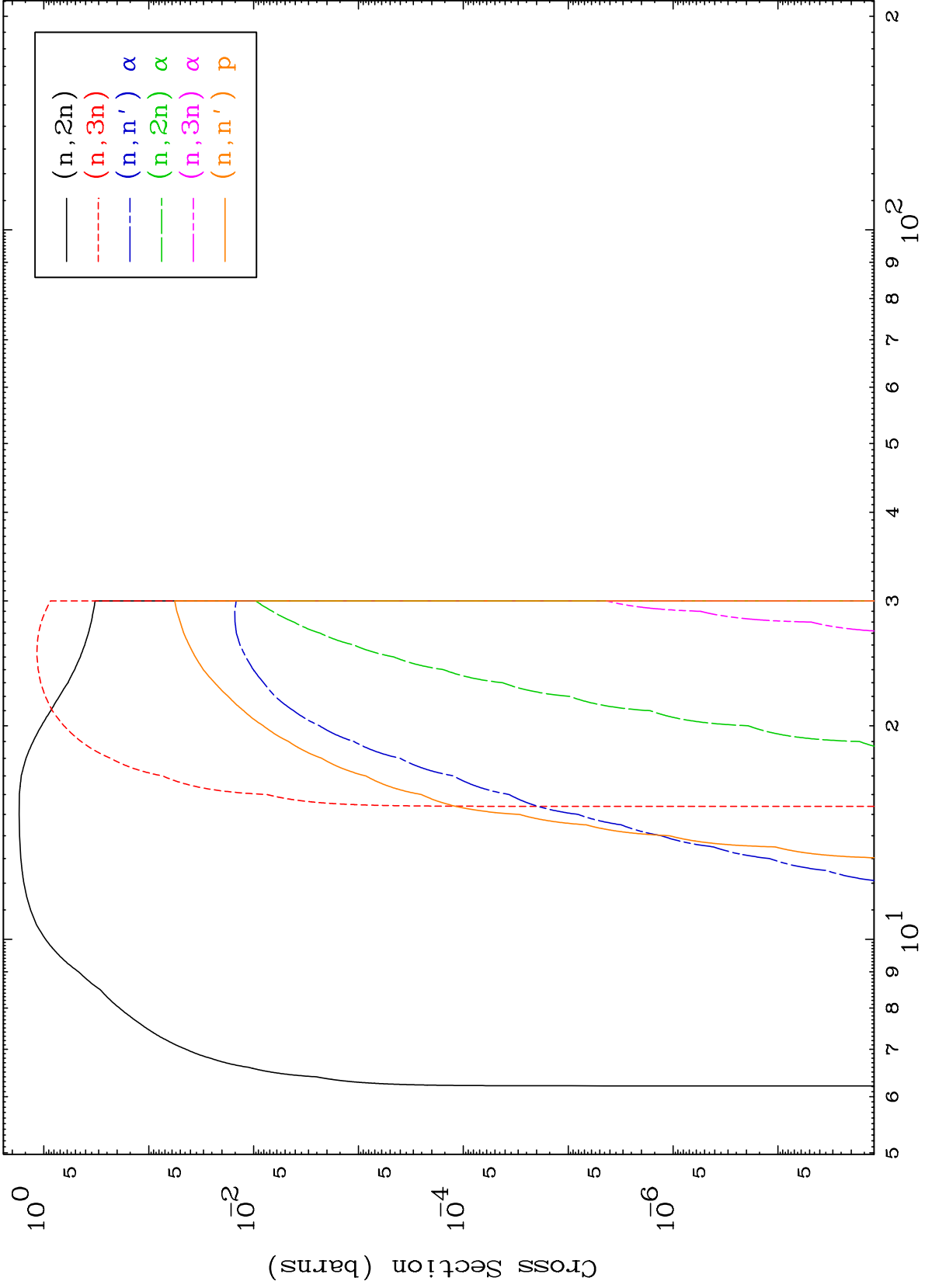
Press Mouse Button to Start



MAT 5053

Neutron Production
293 Kelvin Cross Sections

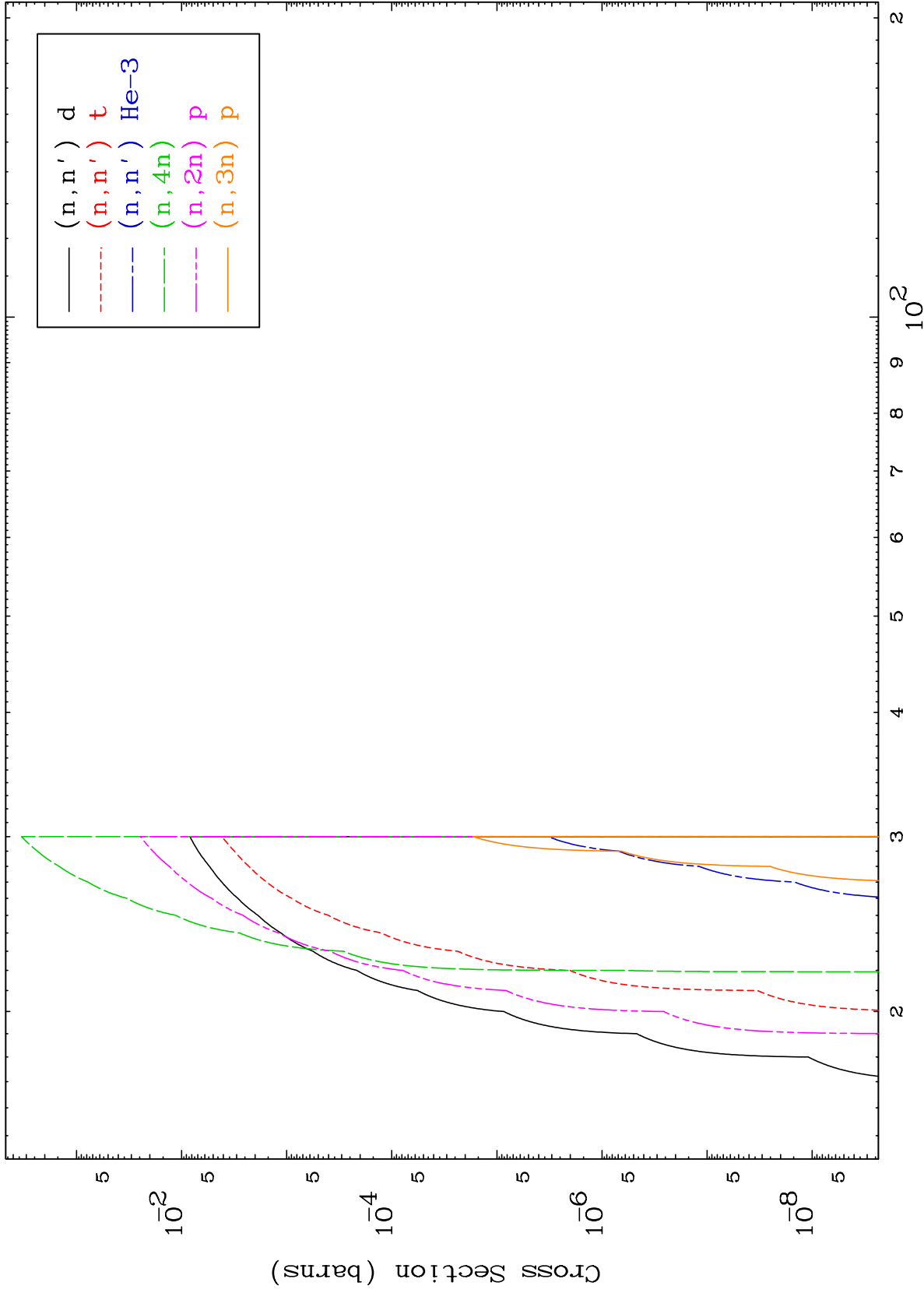
50-Sn-121

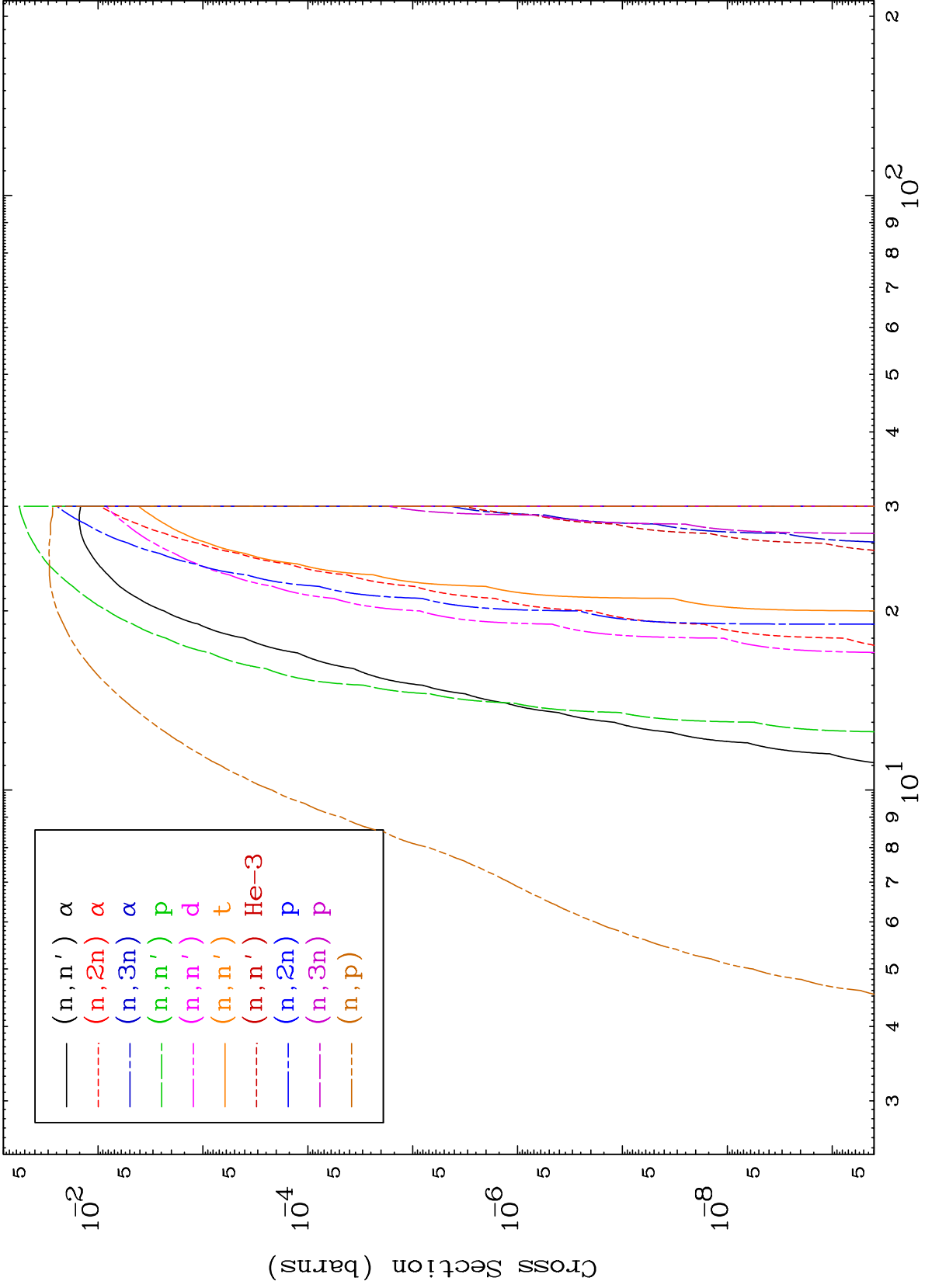


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Incident Energy (MeV)

50-Sn-121

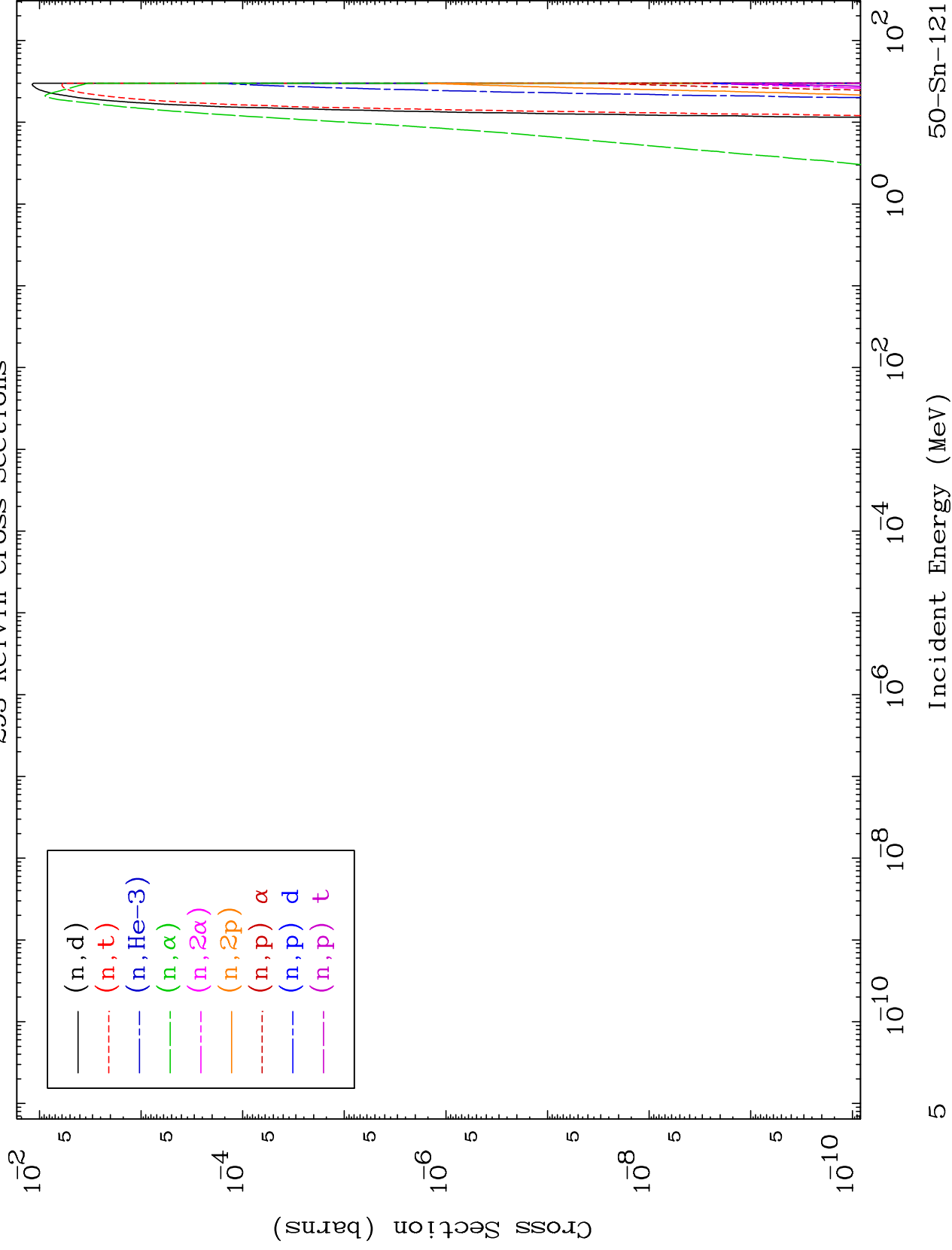




MAT 5053

Charged Particle
293 Kelvin Cross Sections

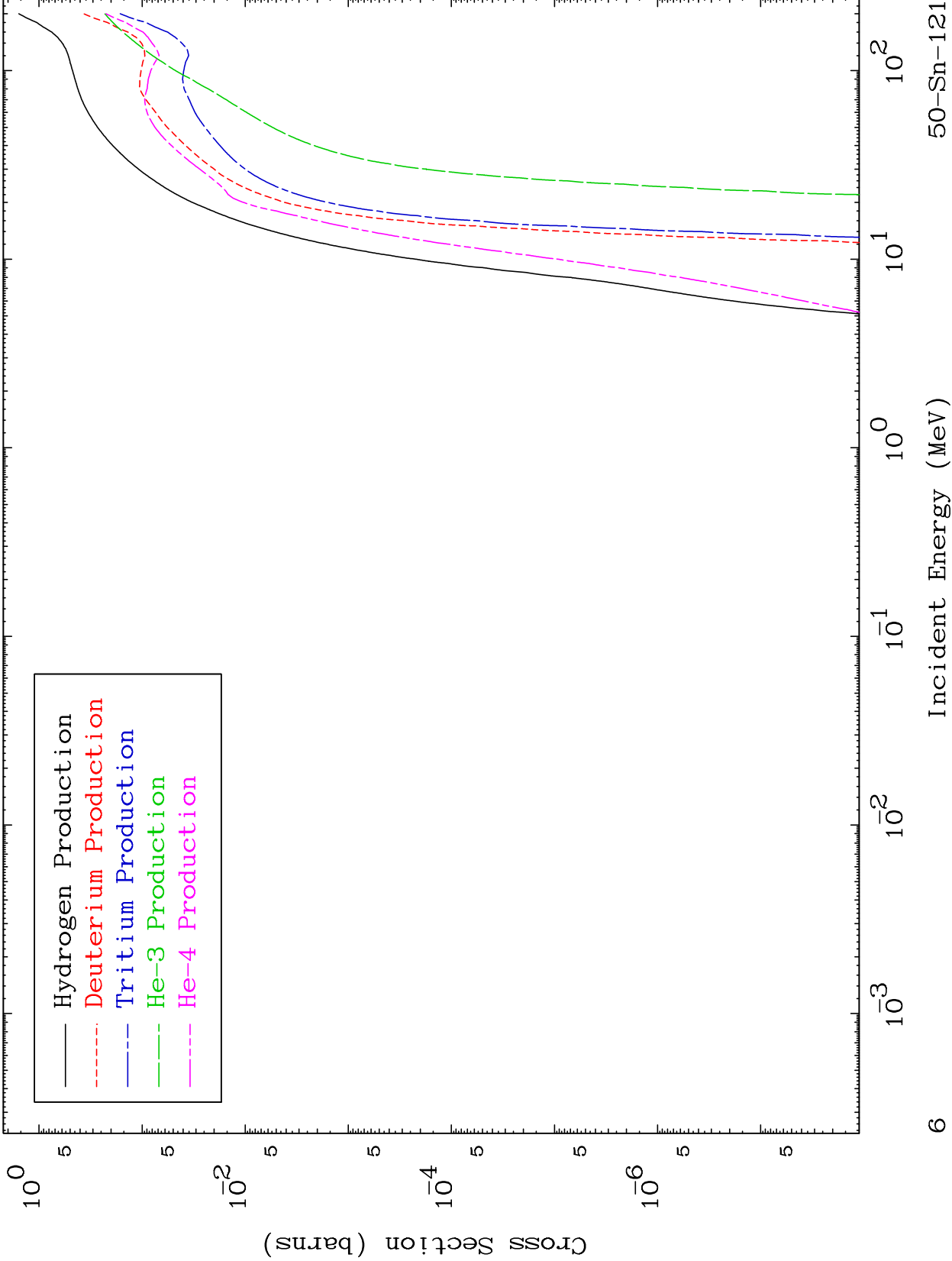
50-Sn-121



MAT 5053

Particle Production
293 Kelvin Cross Sections

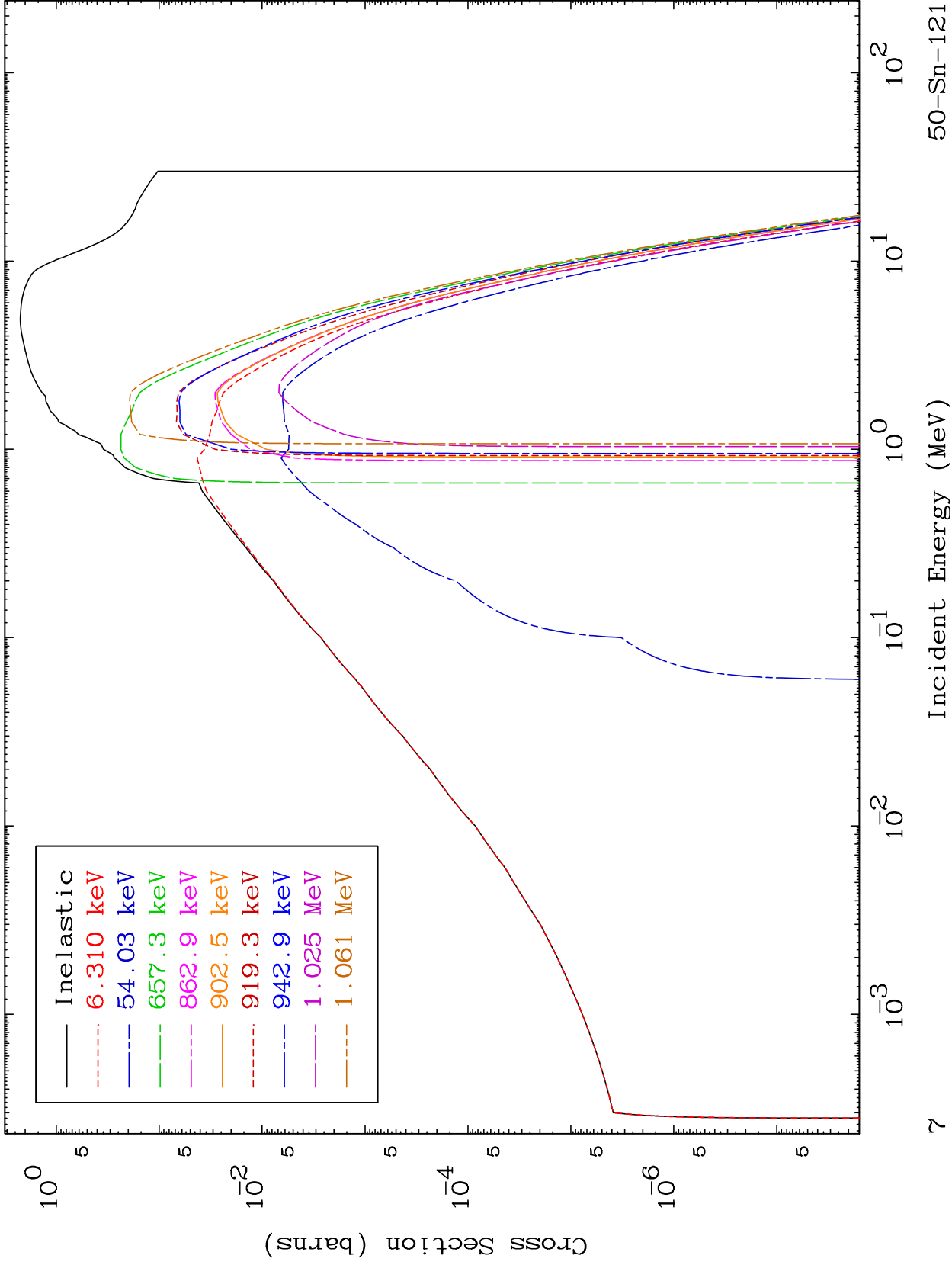
50-Sn-121



MAT 5053

293 Kelvin Cross Sections
(n,n') Level

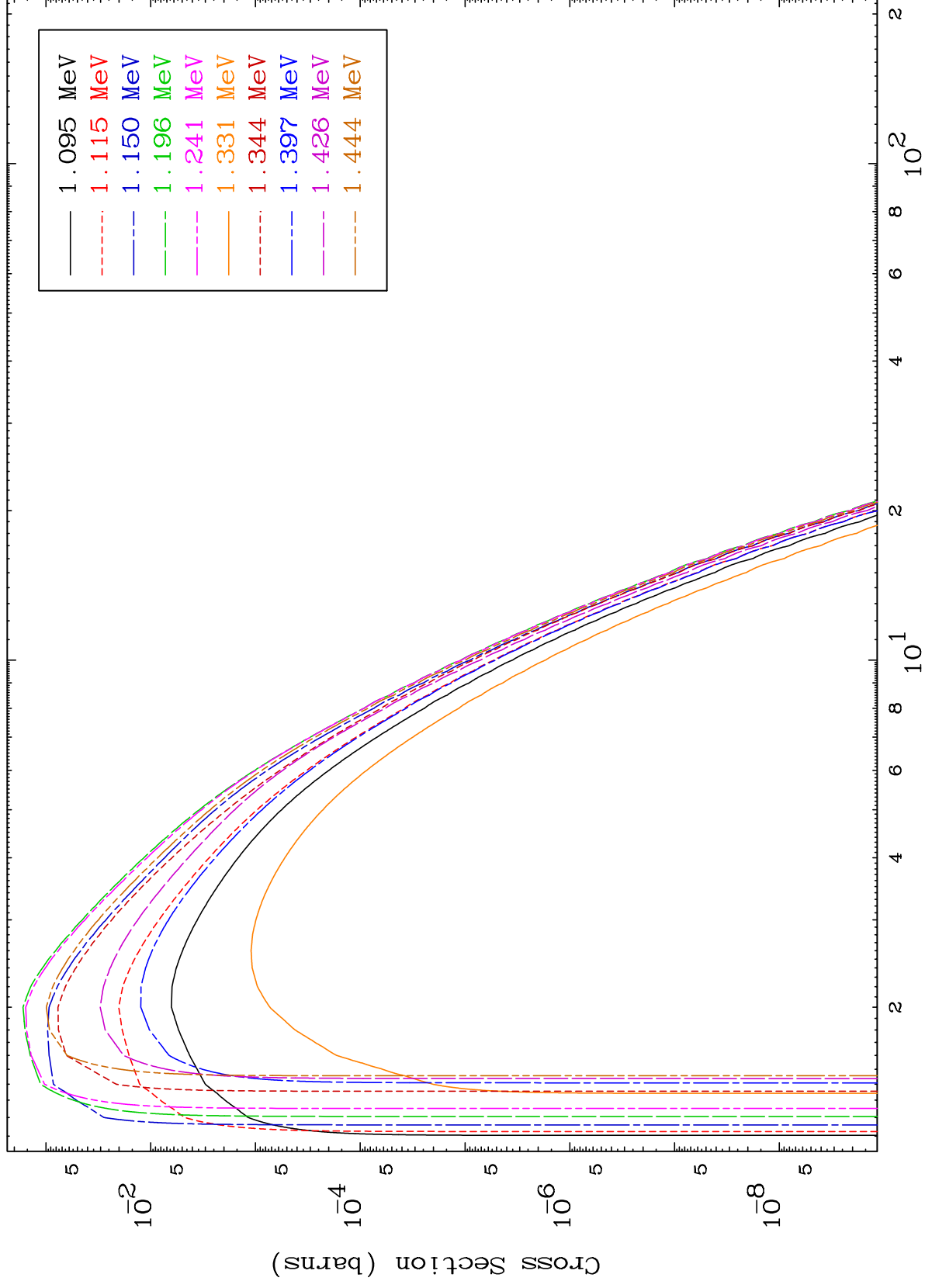
50-Sn-121



MAT 5053

(n,n') Level
293 Kelvin Cross Sections

50-Sn-121



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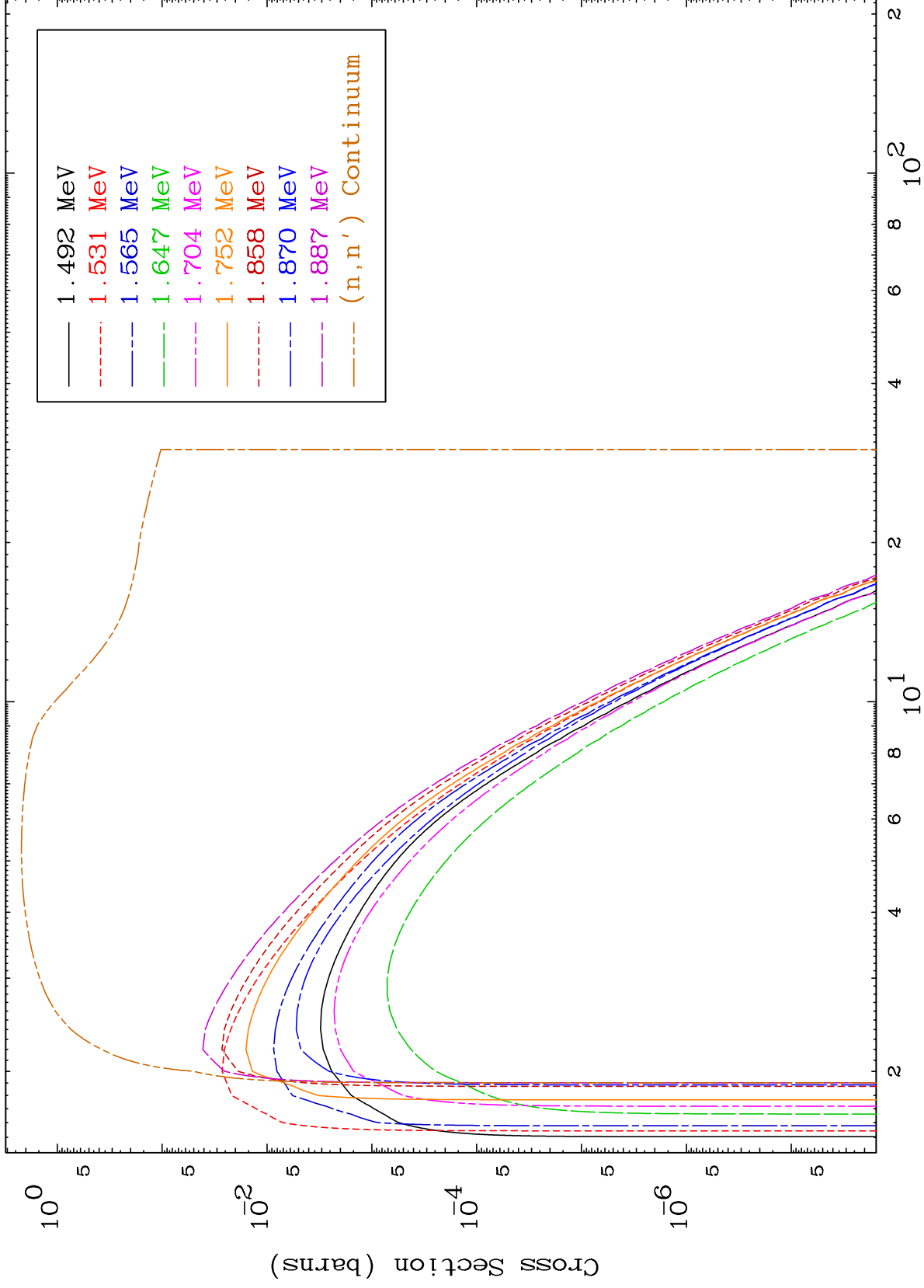
Incident Energy (MeV)

50-Sn-121

MAT 5053

(n,n') Level
293 Kelvin Cross Sections

50-Sn-121



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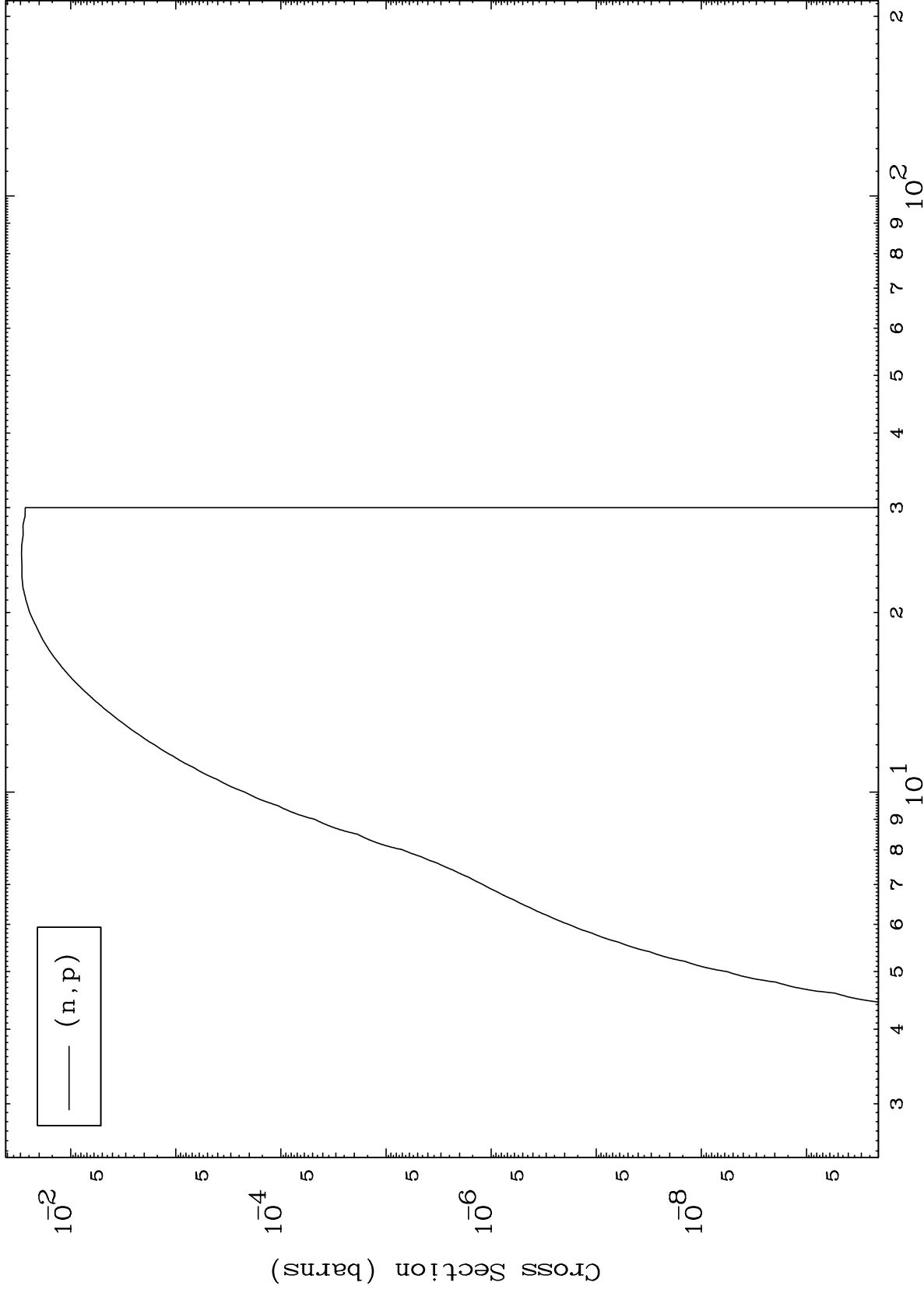
Incident Energy (MeV)

50-Sn-121

MAT 5053

(n,p) Levels
293 Kelvin Cross Sections

50-Sn-121



10

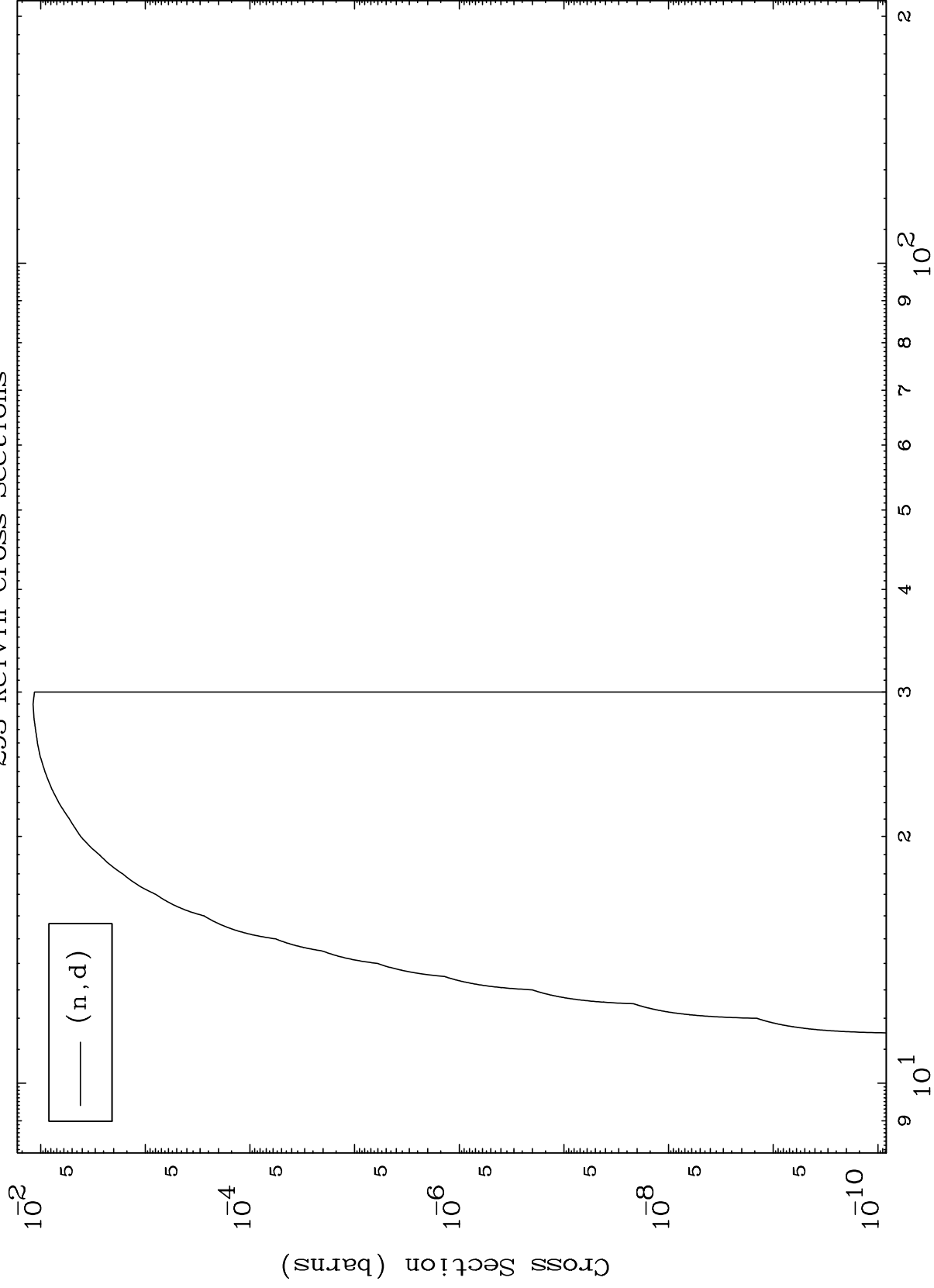
Incident Energy (MeV)

50-Sn-121

MAT 5053

(n,d) Levels
293 Kelvin Cross Sections

50-Sn-121



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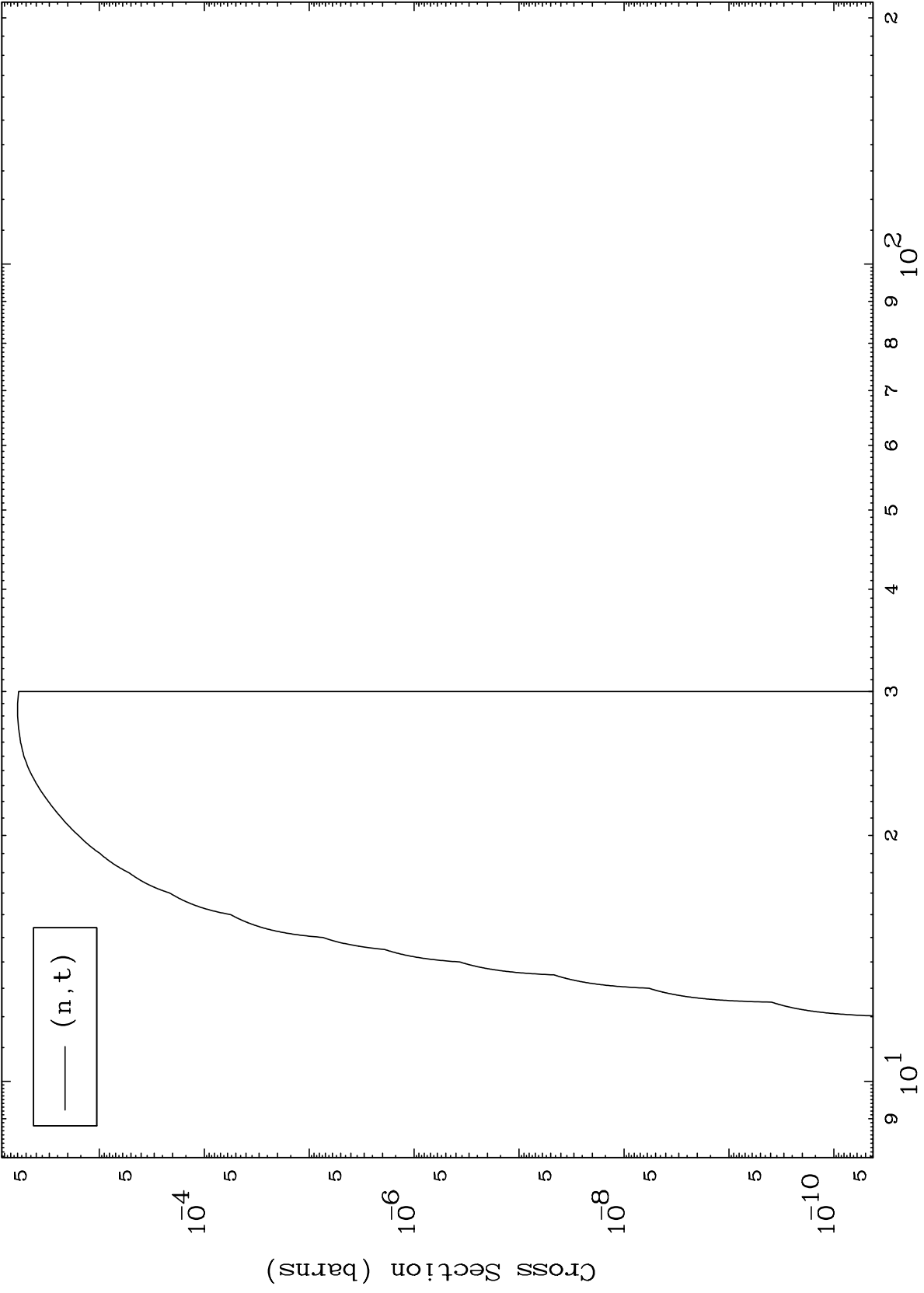
Incident Energy (MeV)

50-Sn-121

MAT 5053

(n,t) Levels
293 Kelvin Cross Sections

50-Sn-121



12

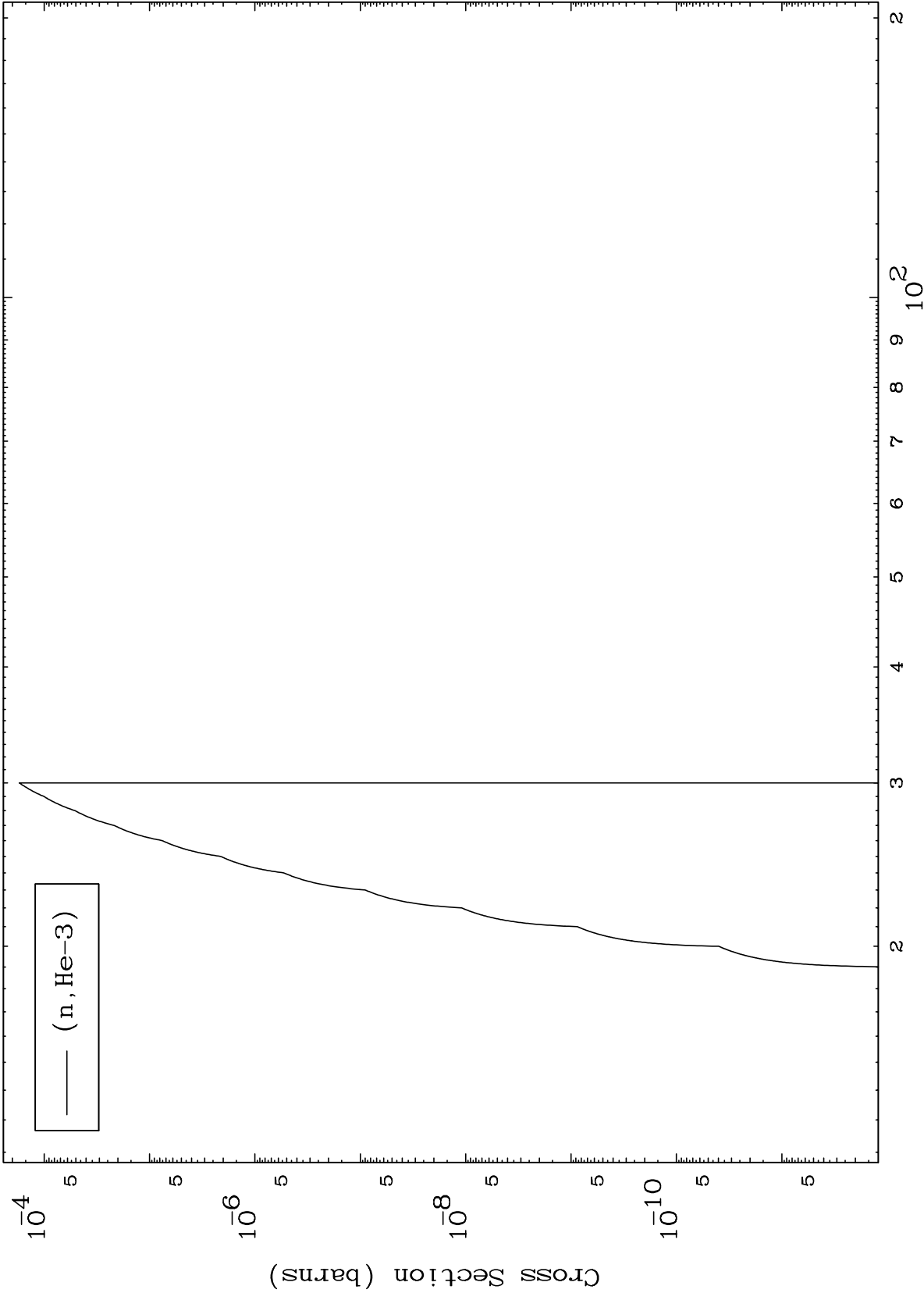
Incident Energy (MeV)

50-Sn-121

MAT 5053

(n,He3) Levels
293 Kelvin Cross Sections

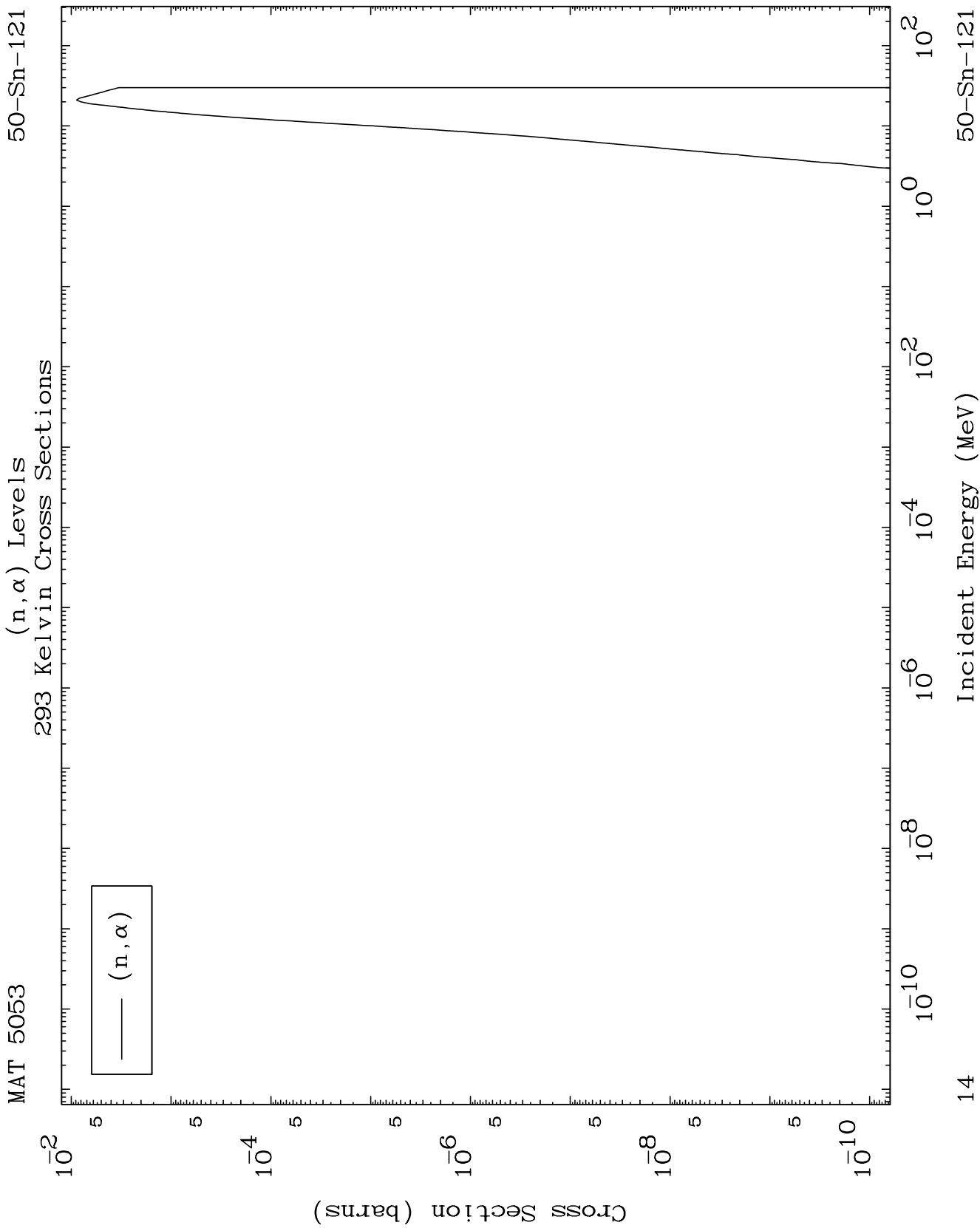
50-Sn-121



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Incident Energy (MeV)

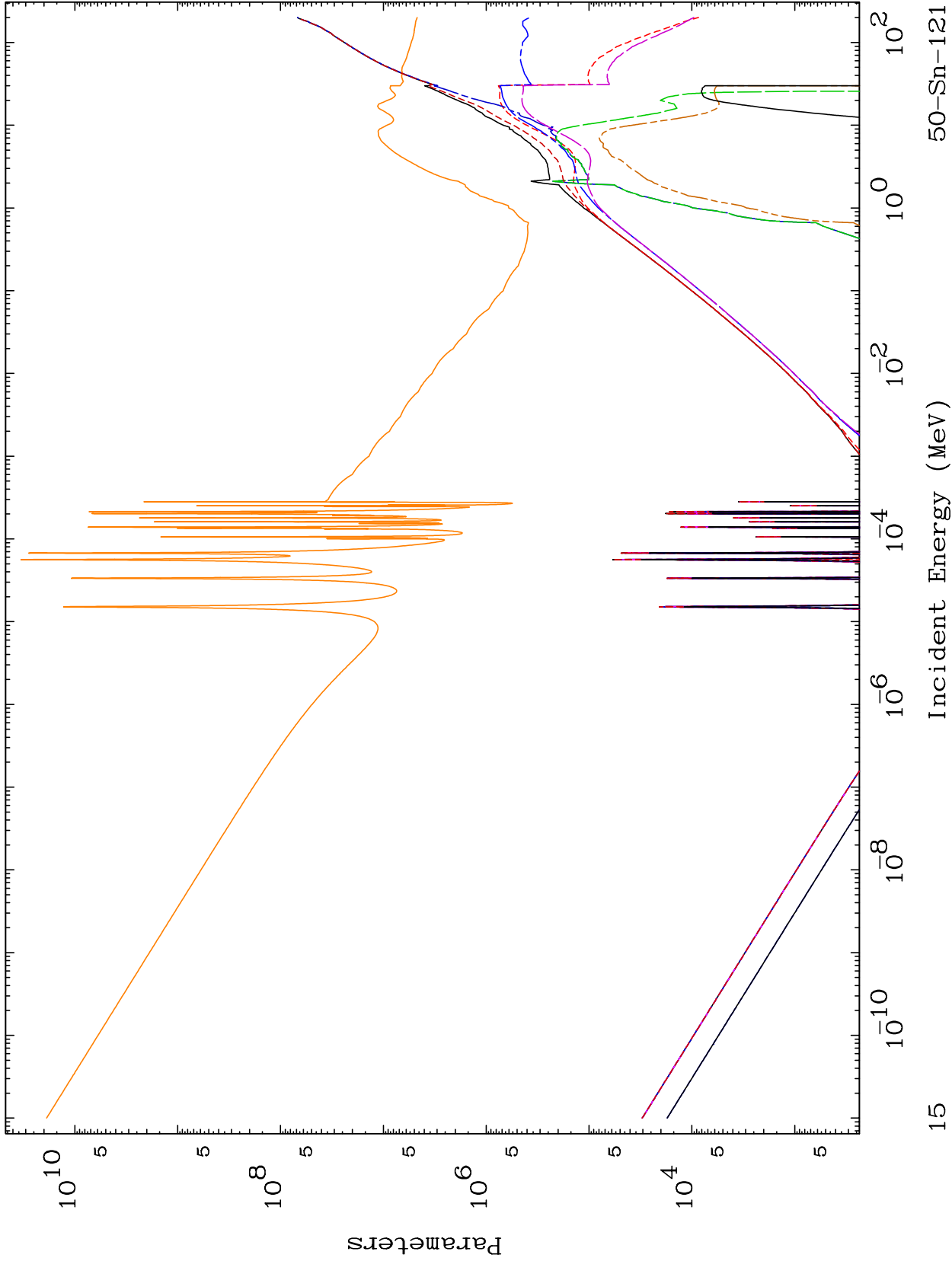
50-Sn-121



MAT 5053

Energy Release
Parameters

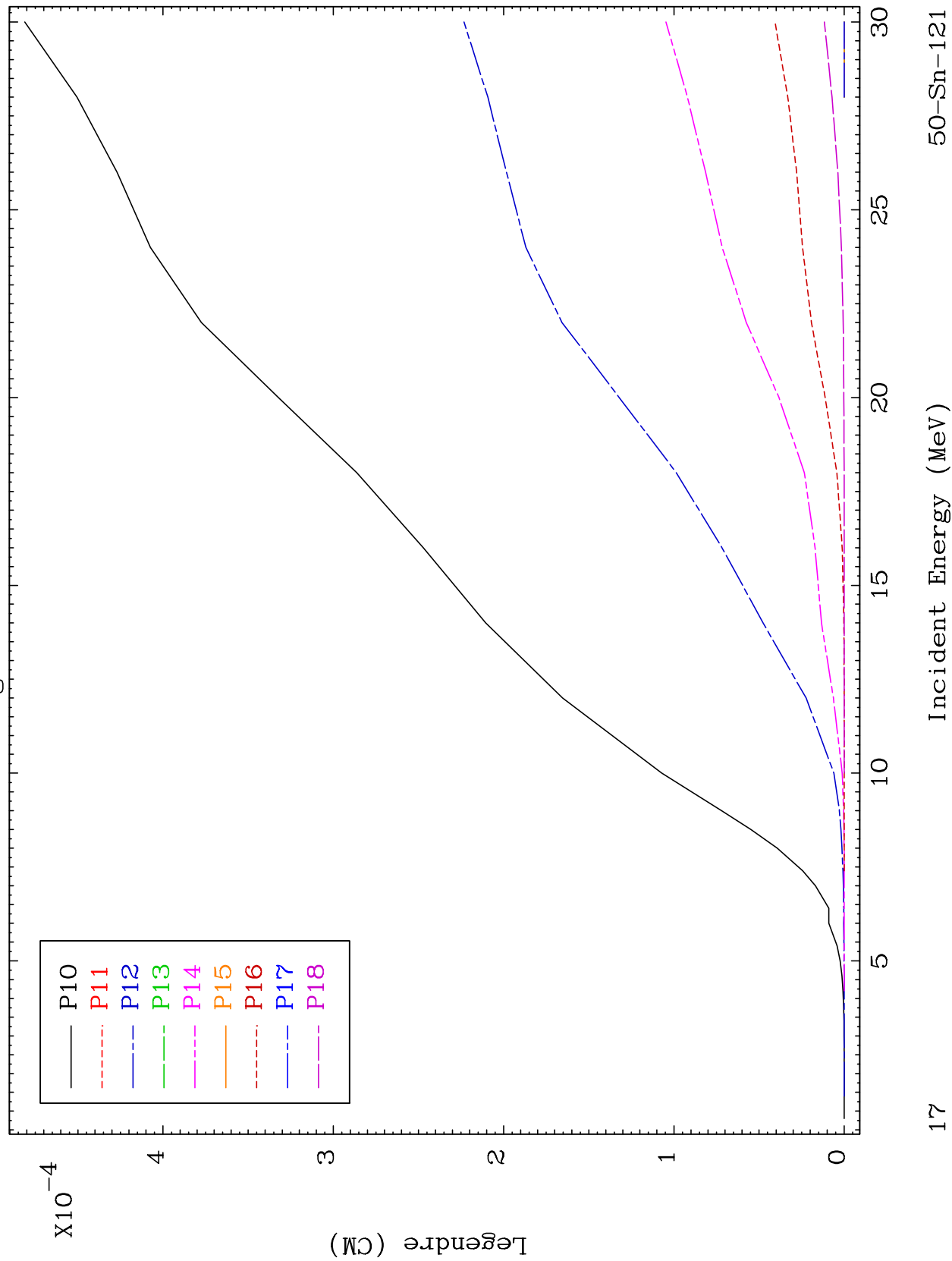
50-Sn-121



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Elastic Legendre Coefficients

50-Sn-121



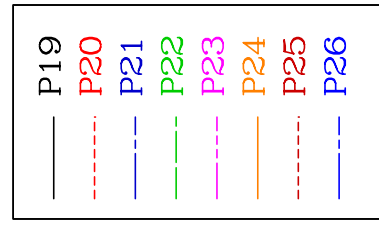
17

50-Sn-121

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Elastic
Legendre Coefficients

50-Sn-121



$\times 10^{-7}$

Legendre (CM)

4

2

0

15

20

25

30

18

Incident Energy (MeV)

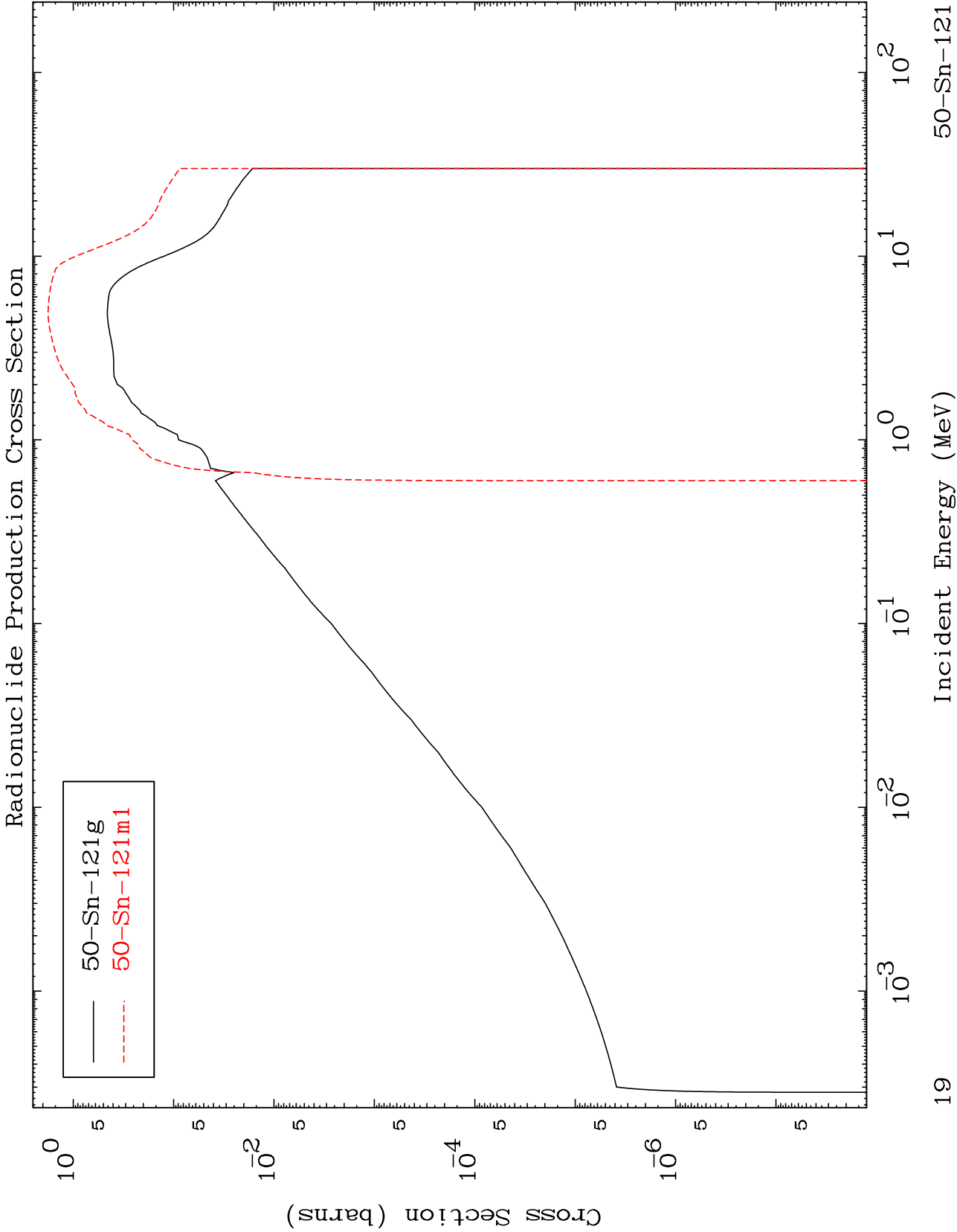
50-Sn-121



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50-Sn-121

Inelastic
Radionuclide Production Cross Section

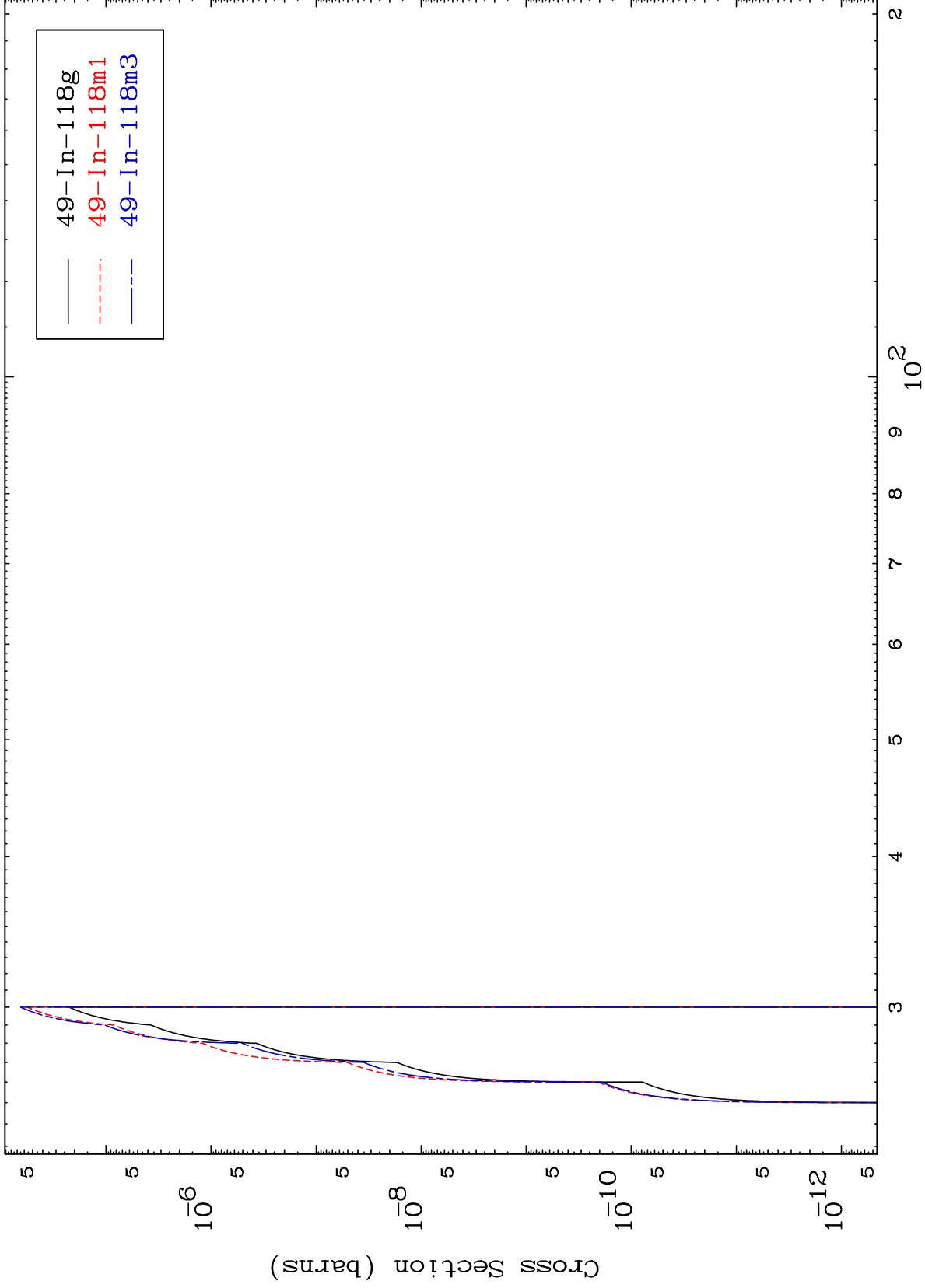


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(n,2n) d

50-Sn-121

Radionuclide Production Cross Section



20

Incident Energy (MeV)

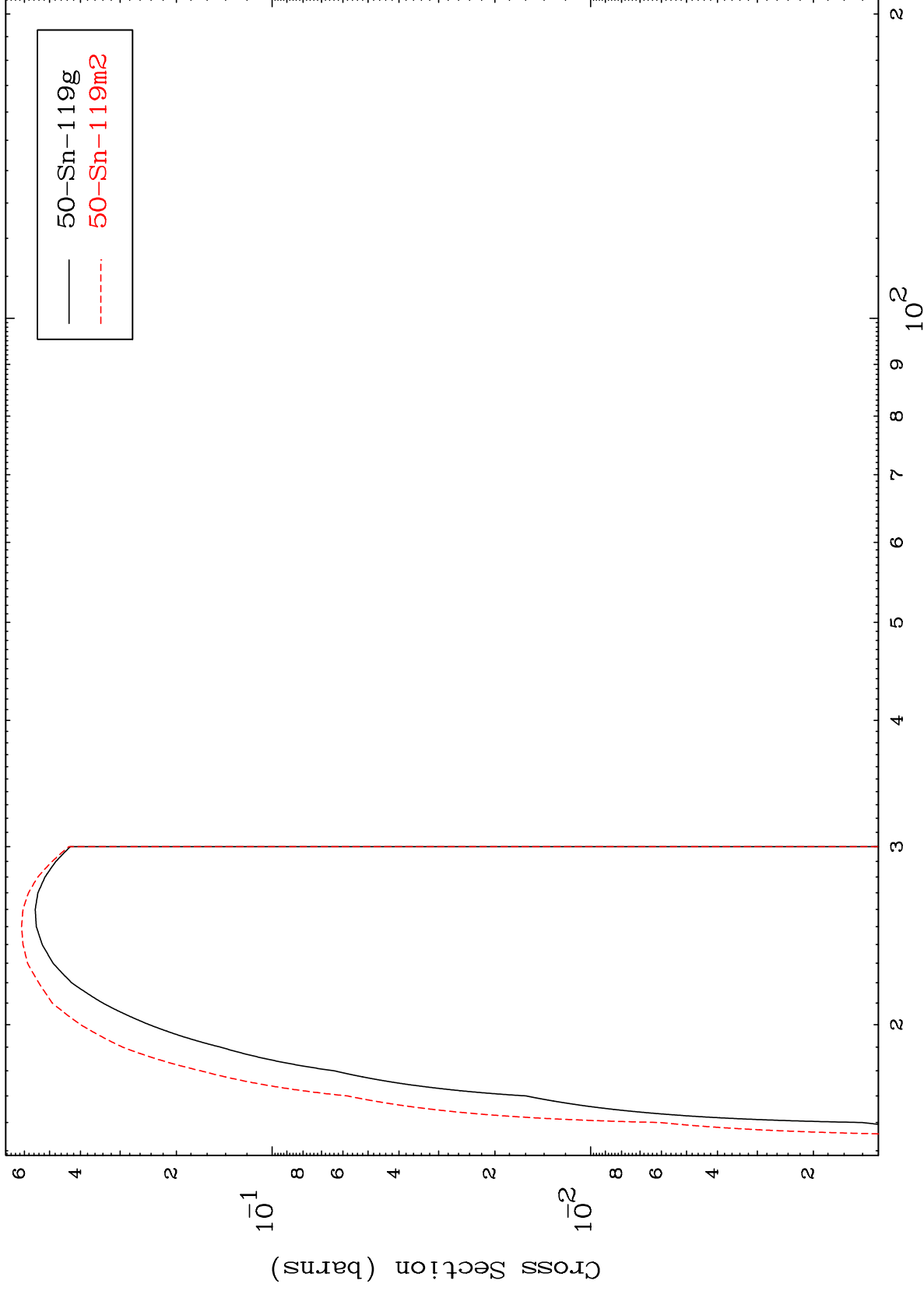
50-Sn-121

MAT 5053

(n,3n)

50-Sn-121

Radionuclide Production Cross Section



21

Incident Energy (MeV)

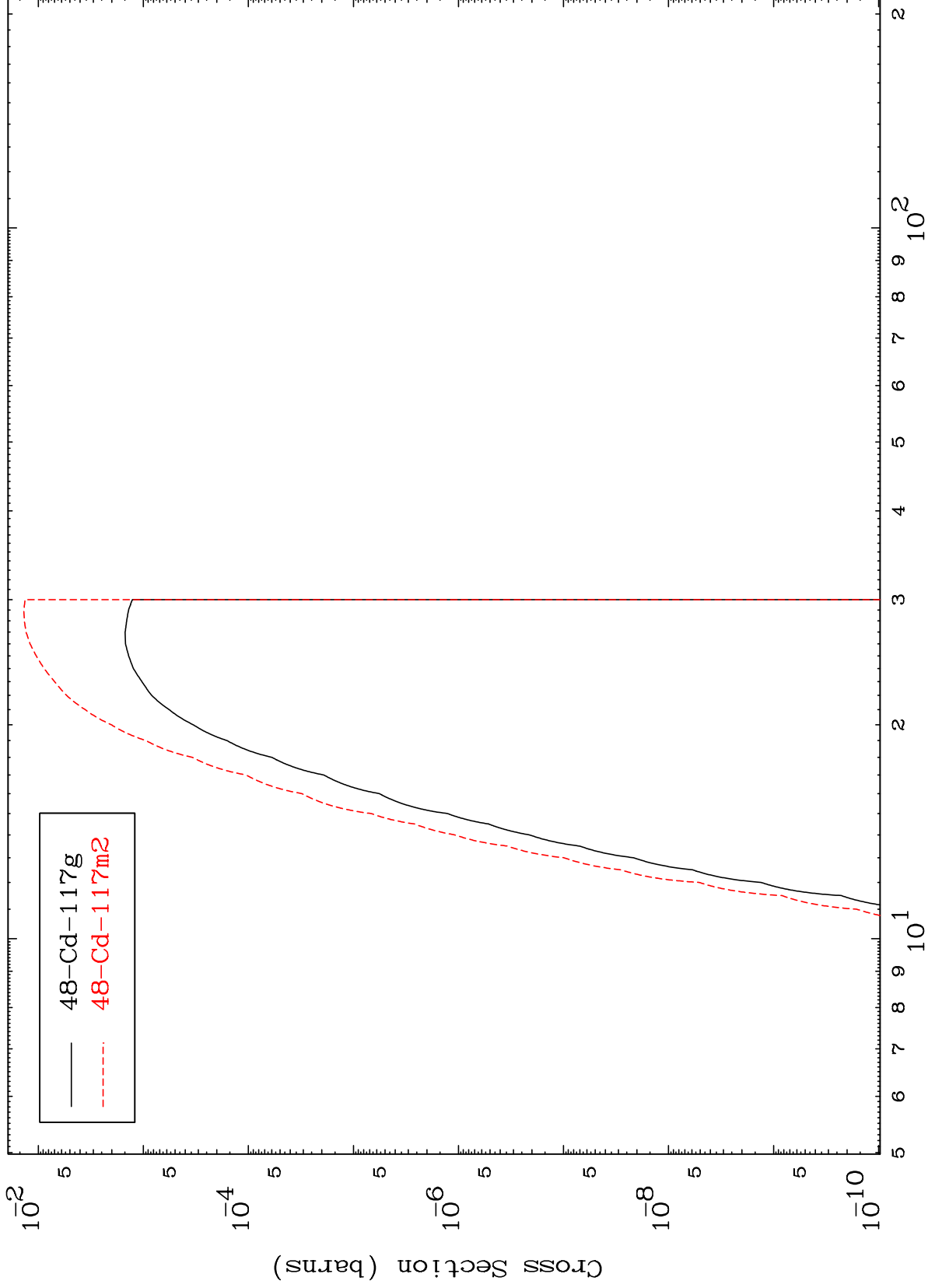
50-Sn-121

MAT 5053

$(n, n') \alpha$

50-Sn-121

Radionuclide Production Cross Section



22

Incident Energy (MeV)

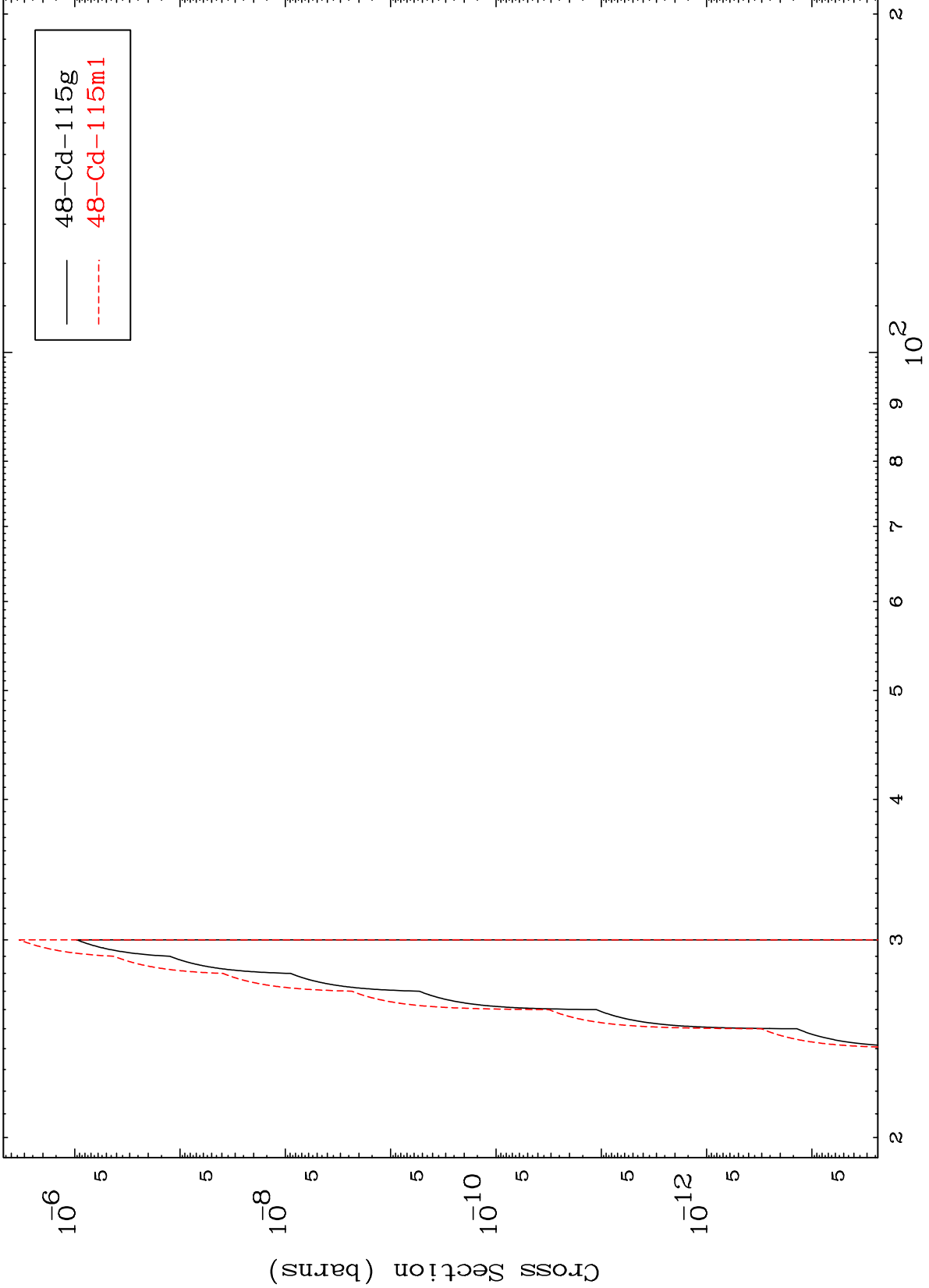
50-Sn-121

MAT 5053

(n,3n) α

50-Sn-121

Radionuclide Production Cross Section

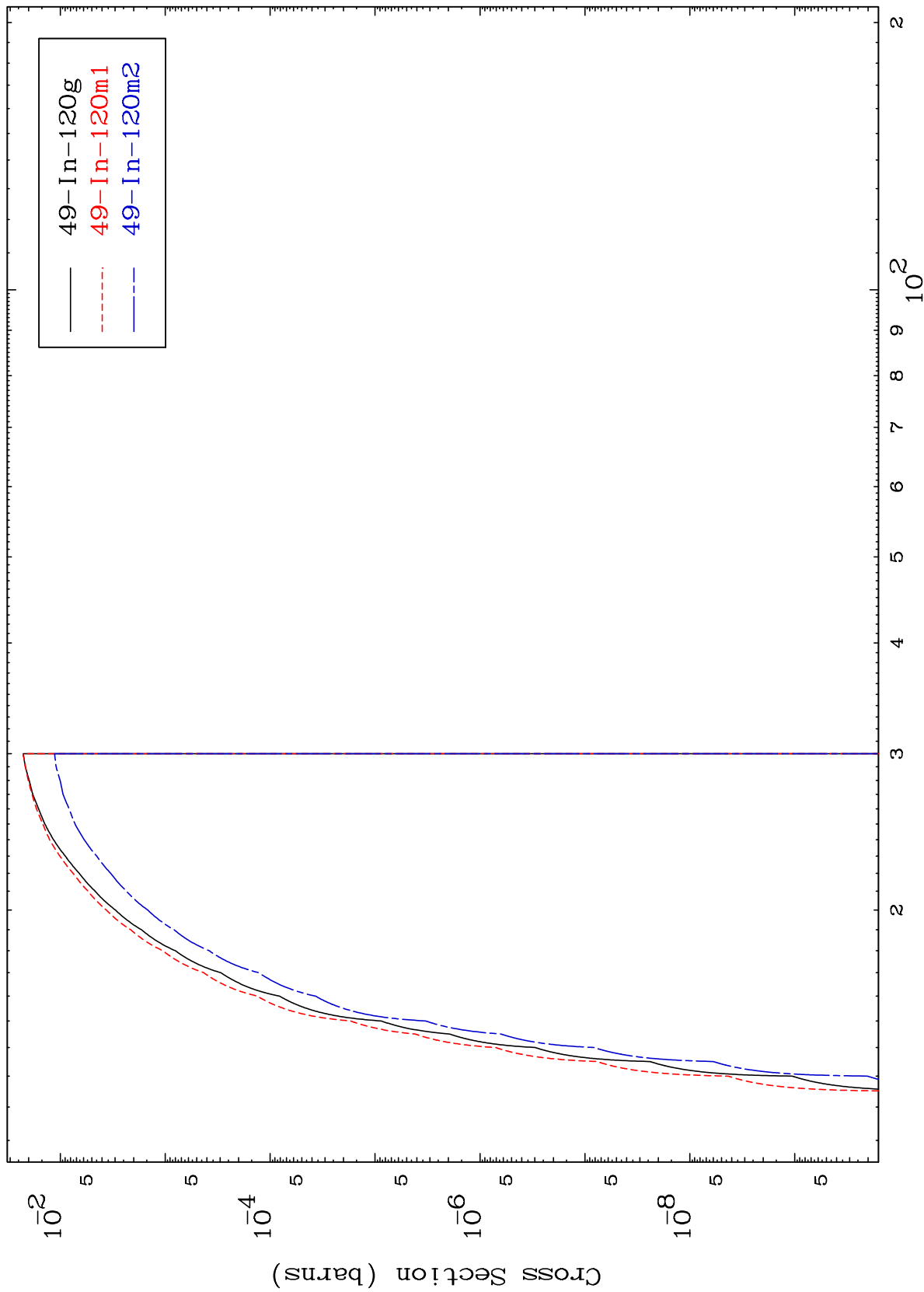


23

Incident Energy (MeV)

50-Sn-121

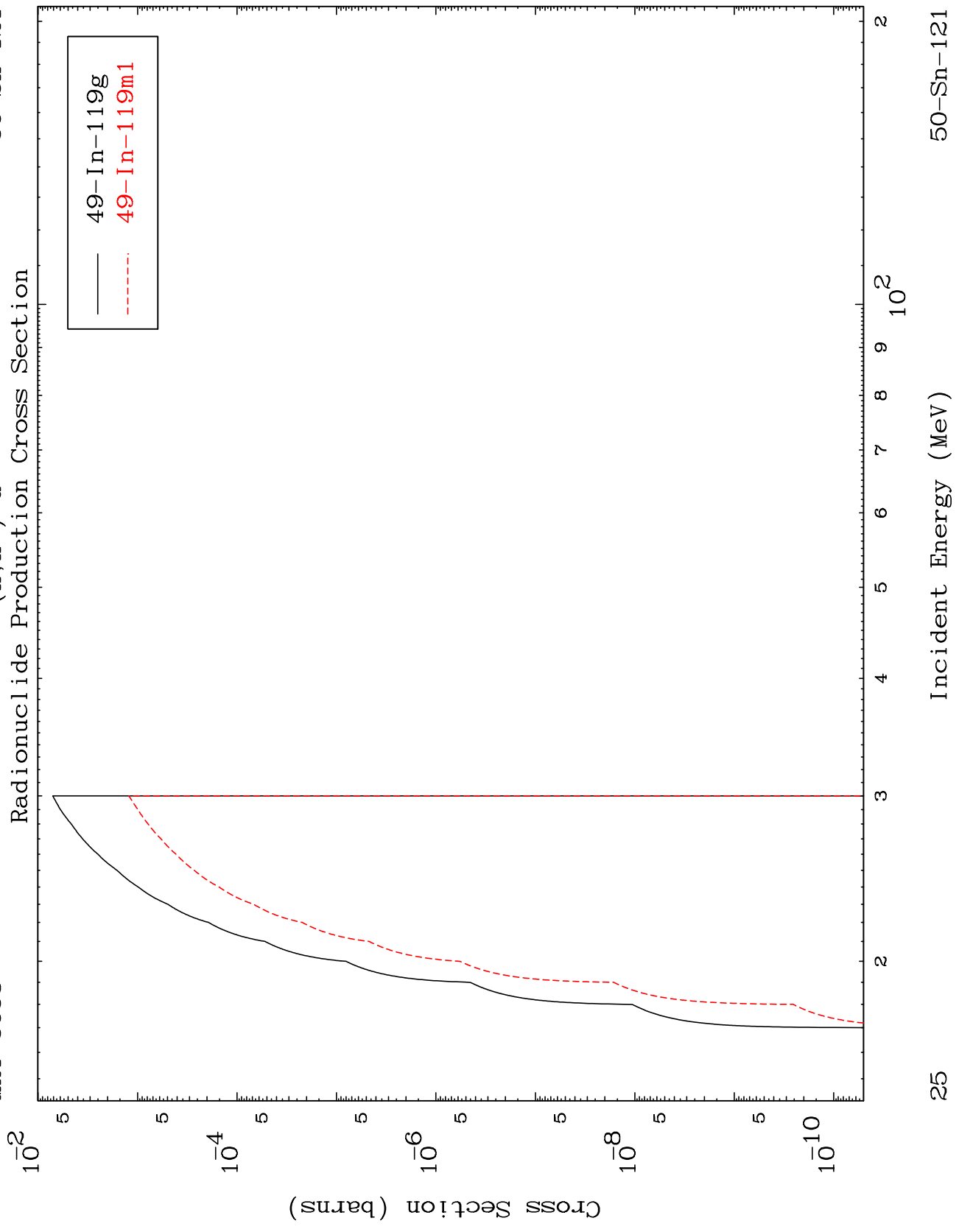
Radionuclide Production Cross Section



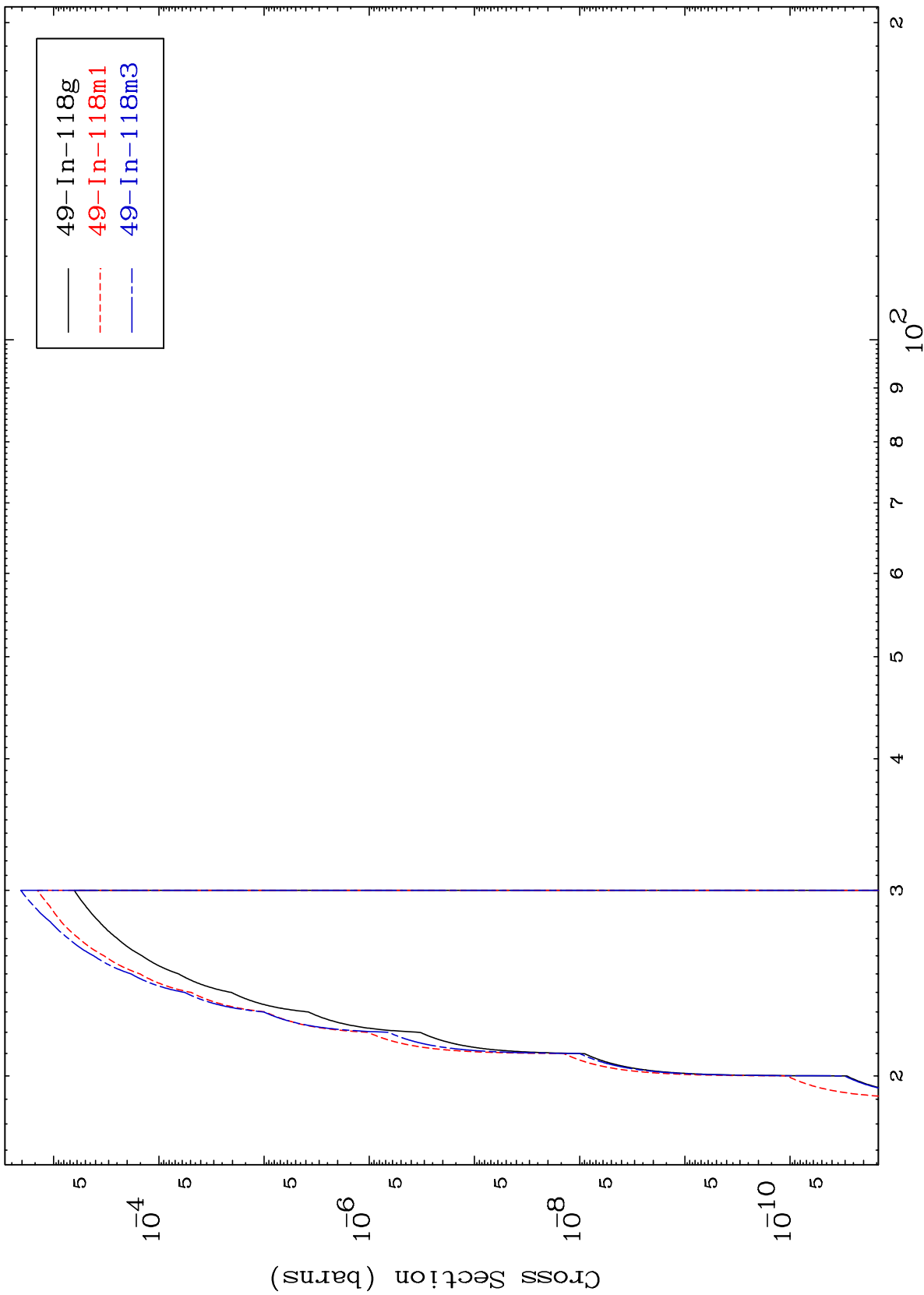
MAT 5053

(n,n') d

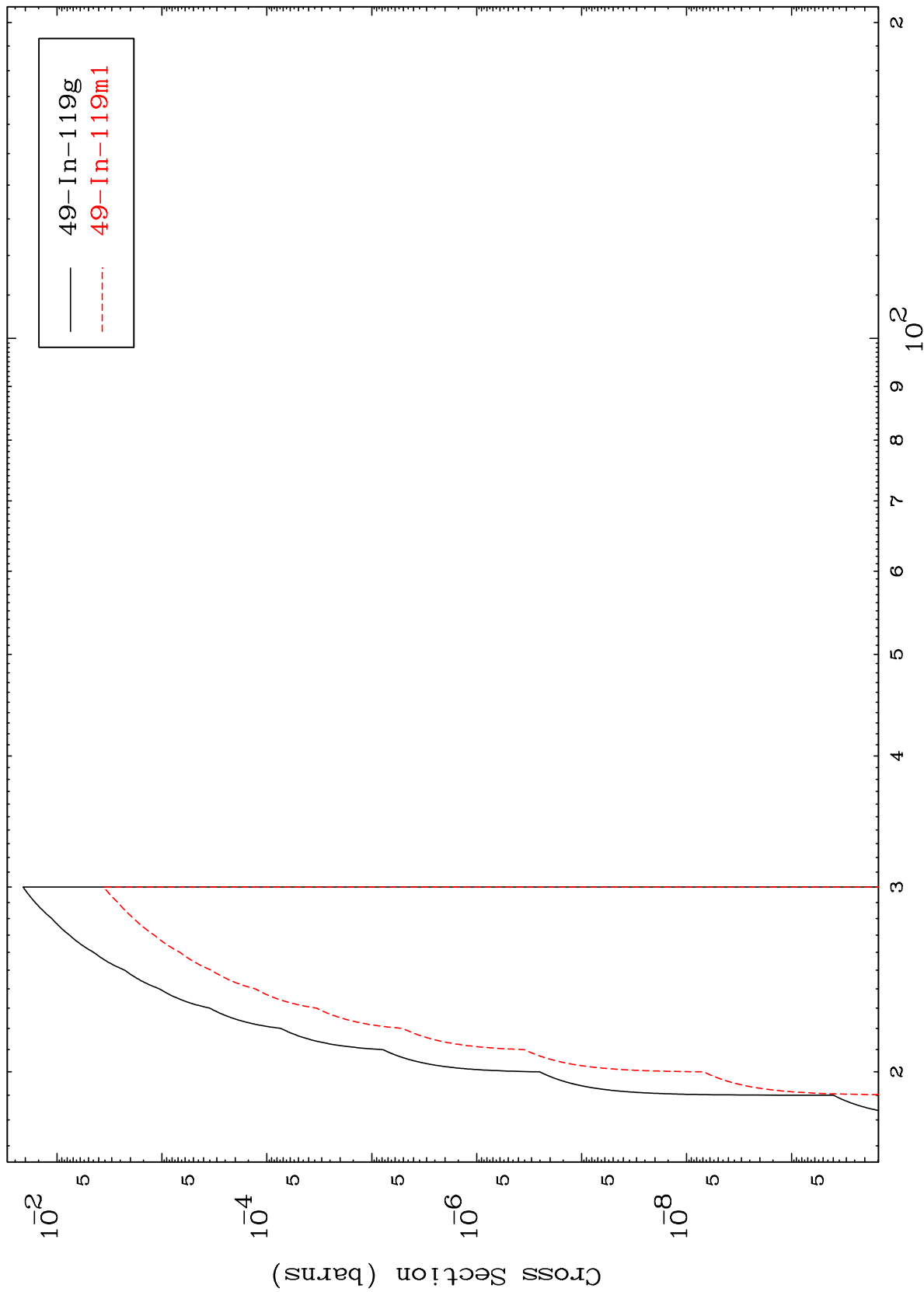
50-Sn-121



Radionuclide Production Cross Section



Radionuclide Production Cross Section

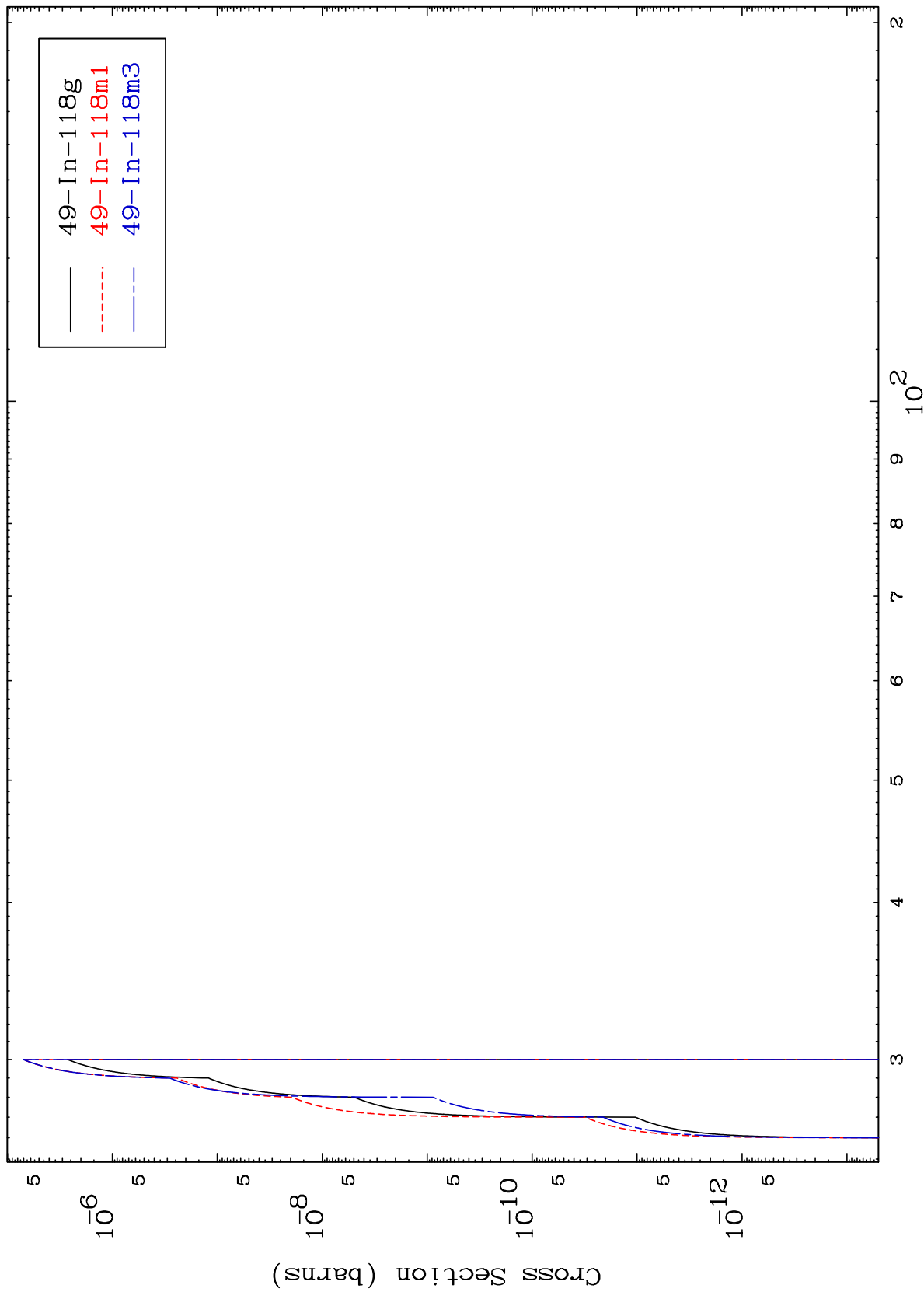


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(n,3n) p

50-Sn-121

Radionuclide Production Cross Section



28

Incident Energy (MeV)

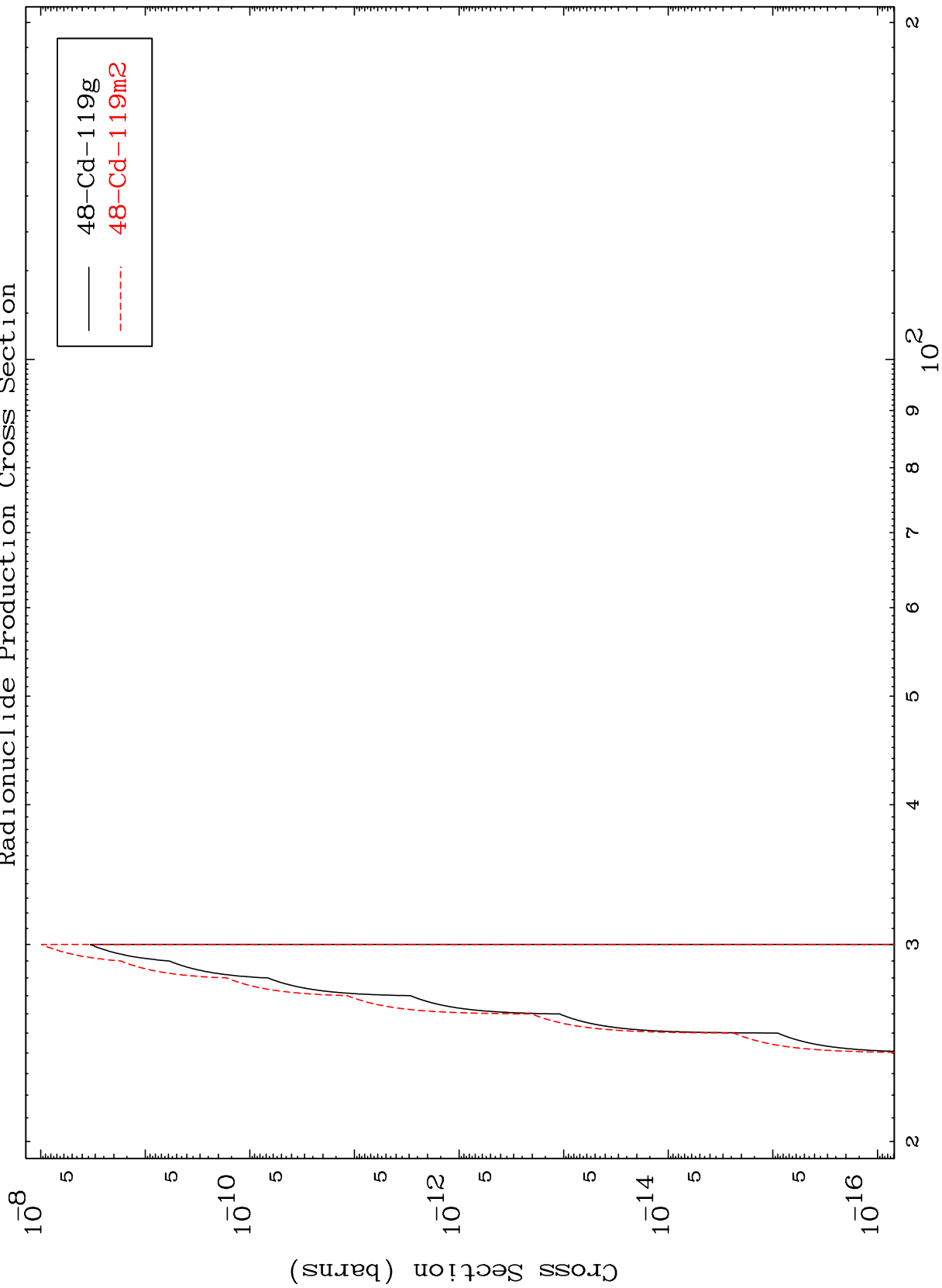
50-Sn-121

MAT 5053

(n,2n) p

50-Sn-121

Radionuclide Production Cross Section



29

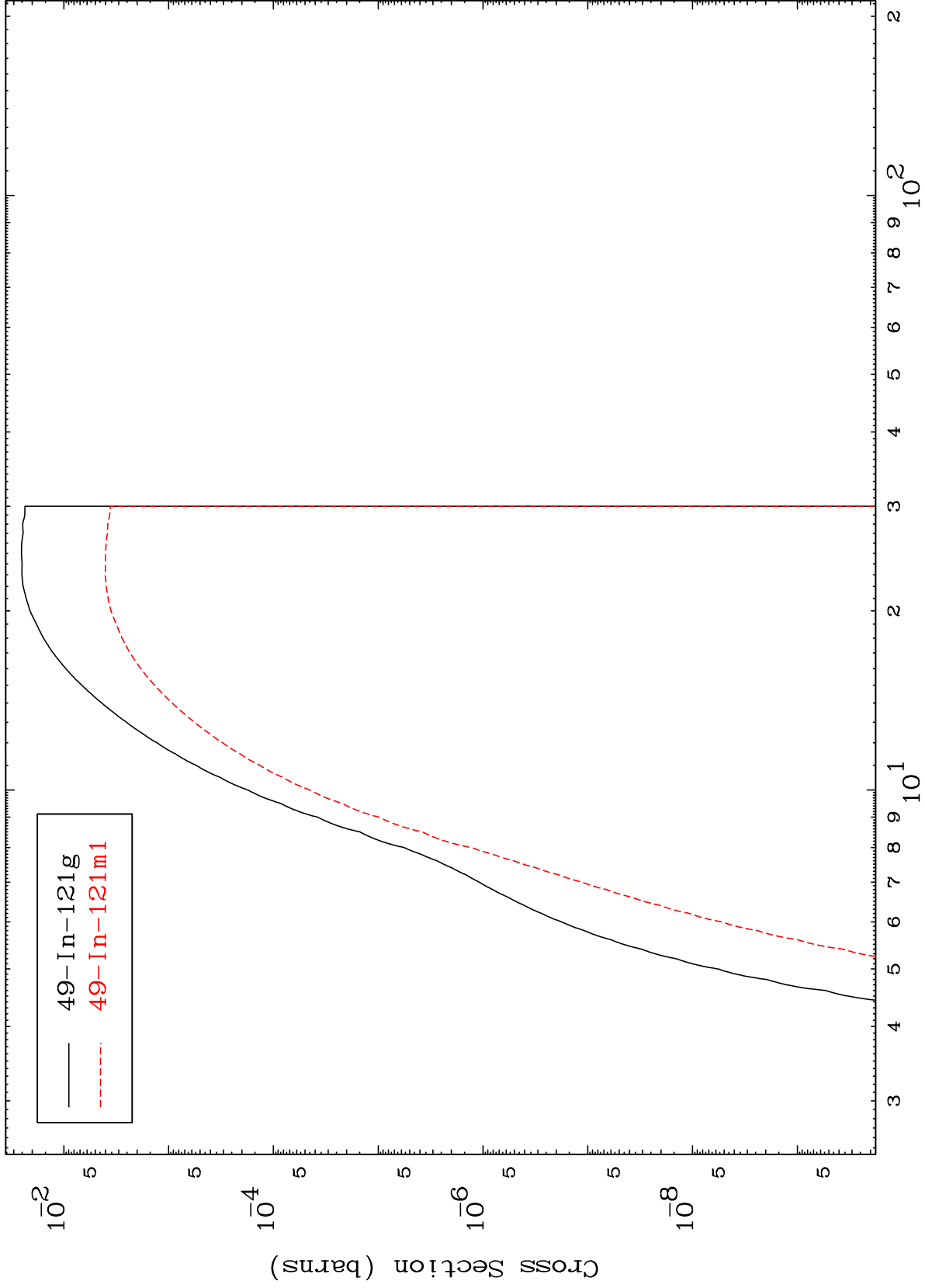
Incident Energy (MeV)

50-Sn-121

MAT 5053

50-Sn-121

(n,p)
Radionuclide Production Cross Section

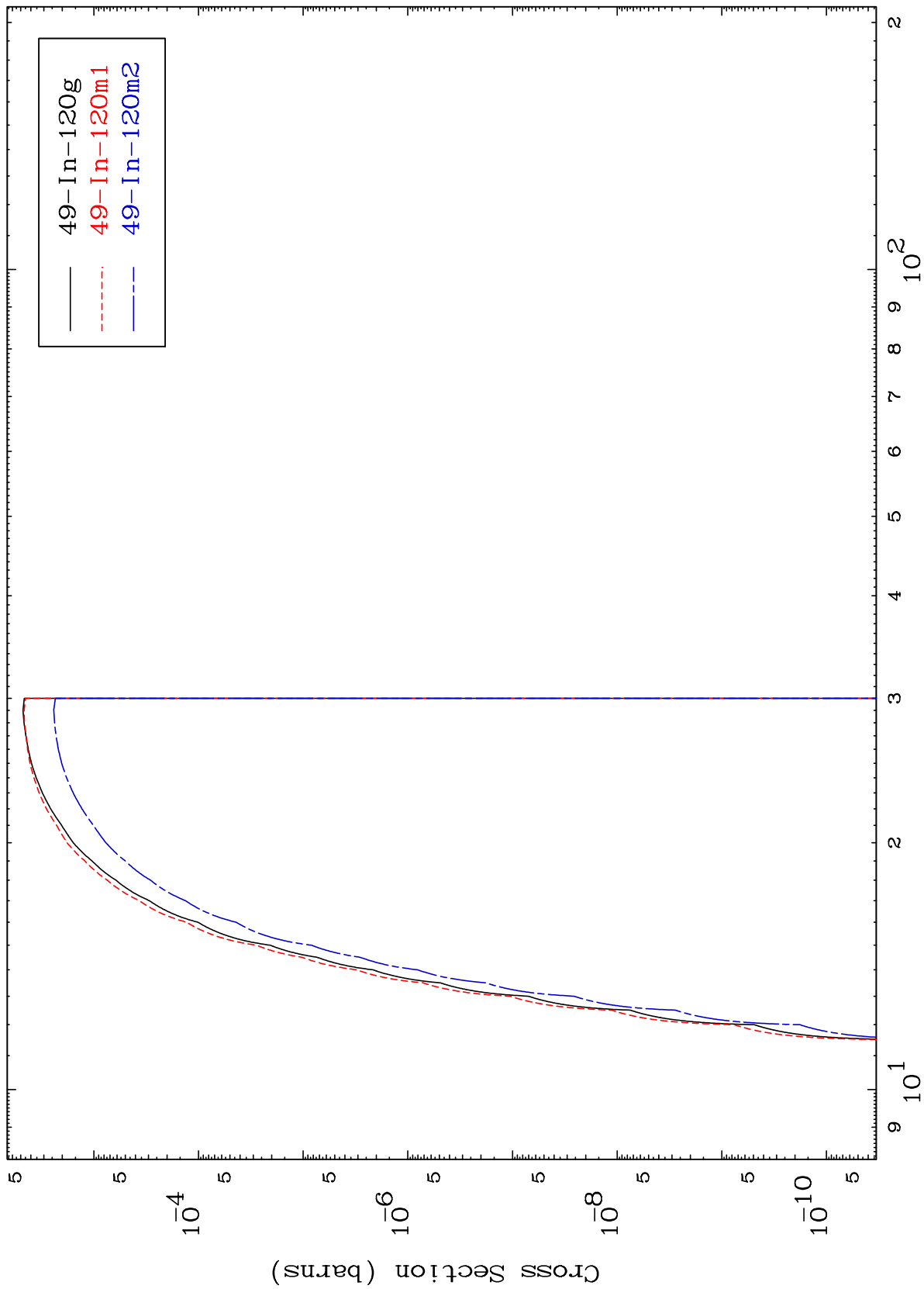


30

Incident Energy (MeV)

50-Sn-121

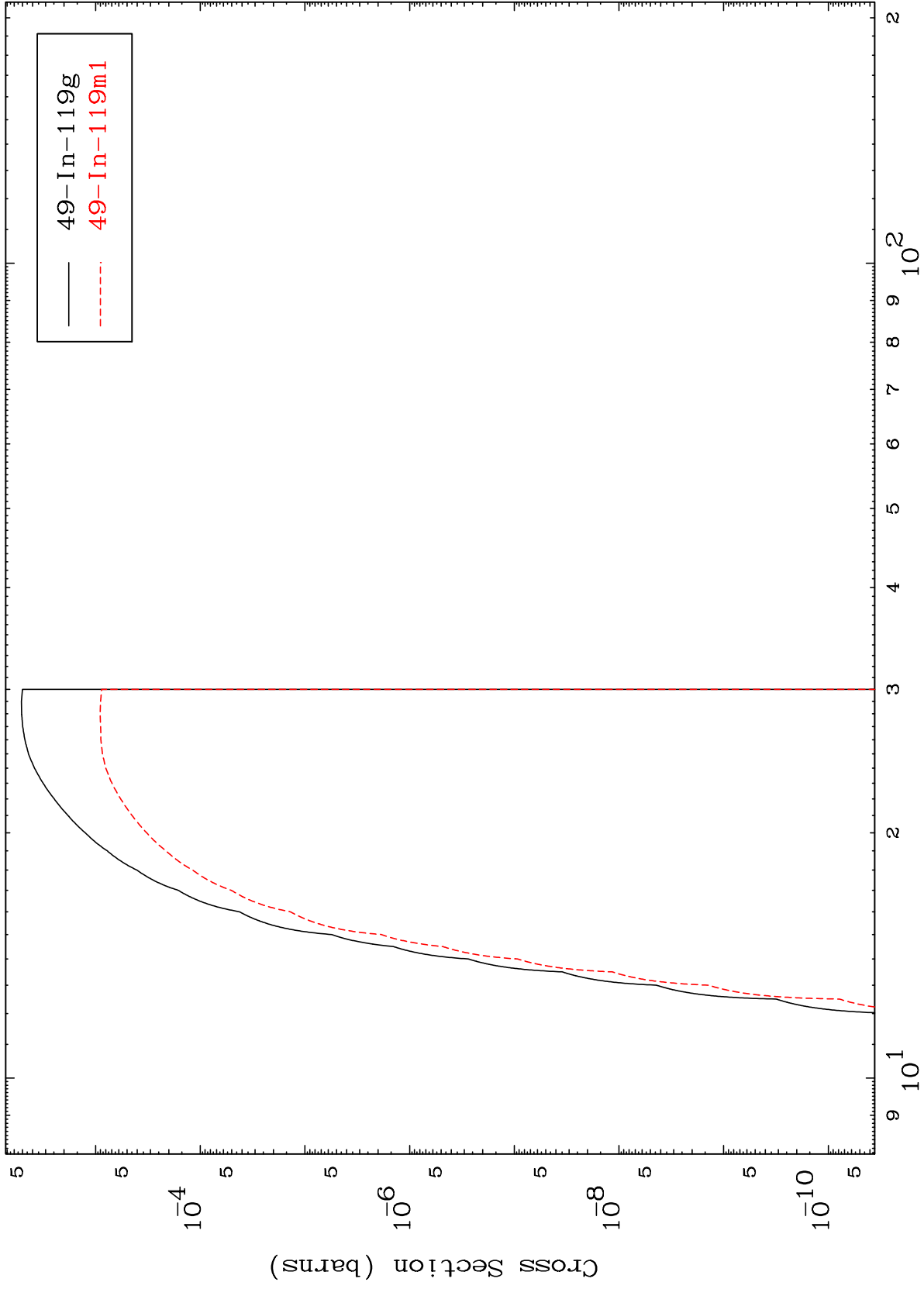
Radionuclide Production Cross Section



MAT 5053

50-Sn-121

(n,t)
Radionuclide Production Cross Section



50-Sn-121

Incident Energy (MeV)

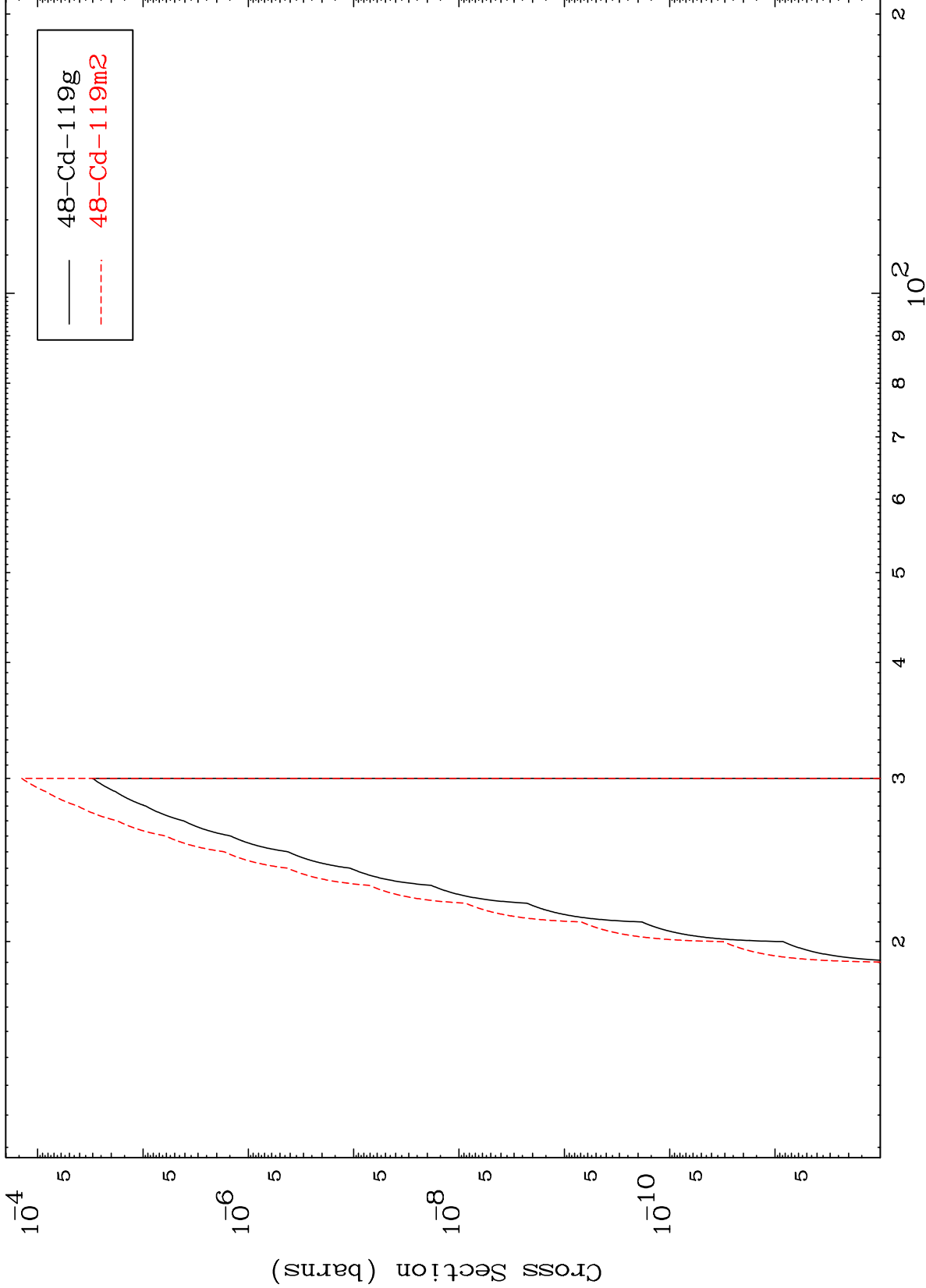
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(n,He-3)

50-Sn-121

Radionuclide Production Cross Section



48-Cd-119g
48-Cd-119m2

33

Incident Energy (MeV)

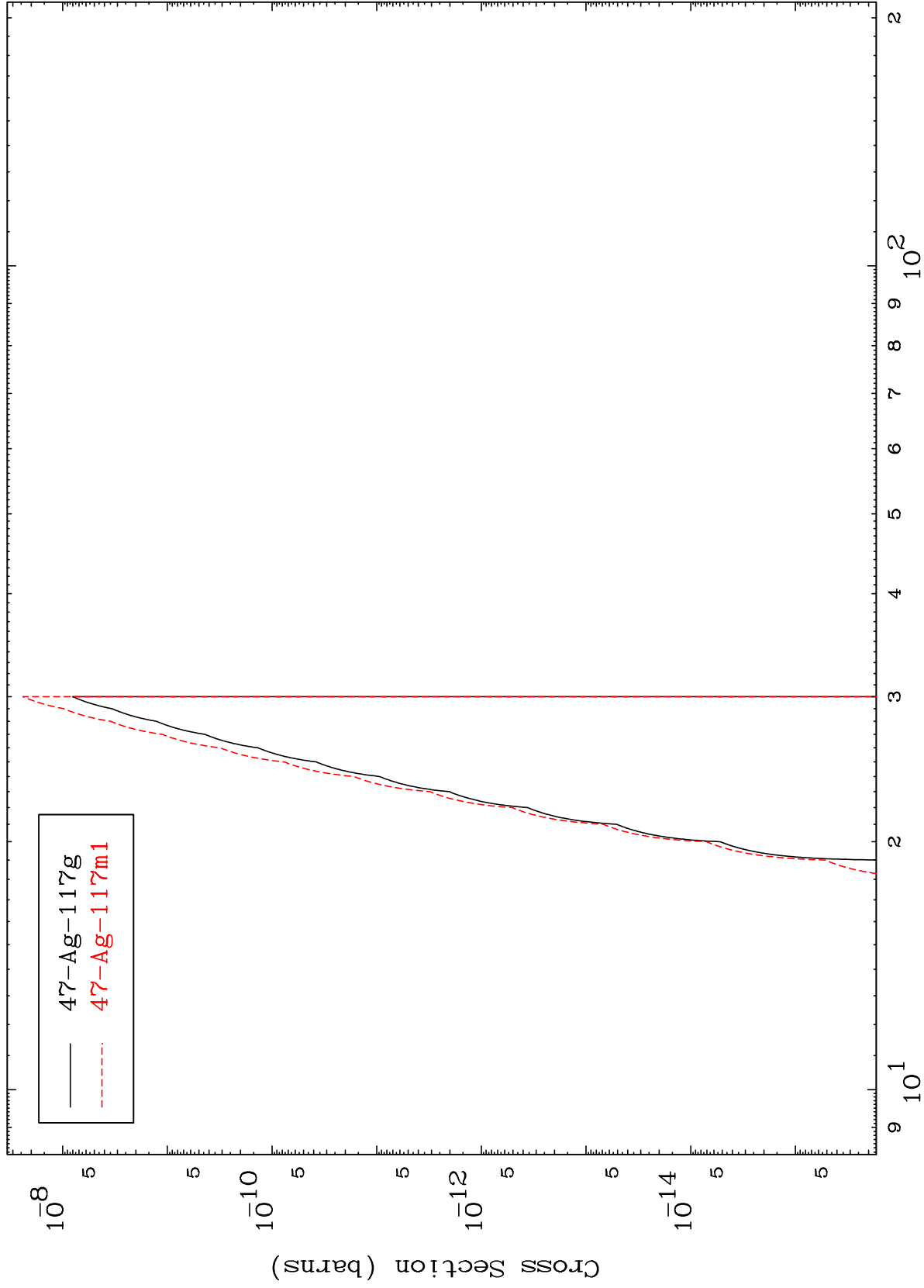
50-Sn-121

MAT 5053

(n,p) α

50-Sn-121

Radionuclide Production Cross Section



34

Incident Energy (MeV)

50-Sn-121

Radionuclide Production Cross Section

